

Program EVALPLOT  
(Version 2015-2)

by

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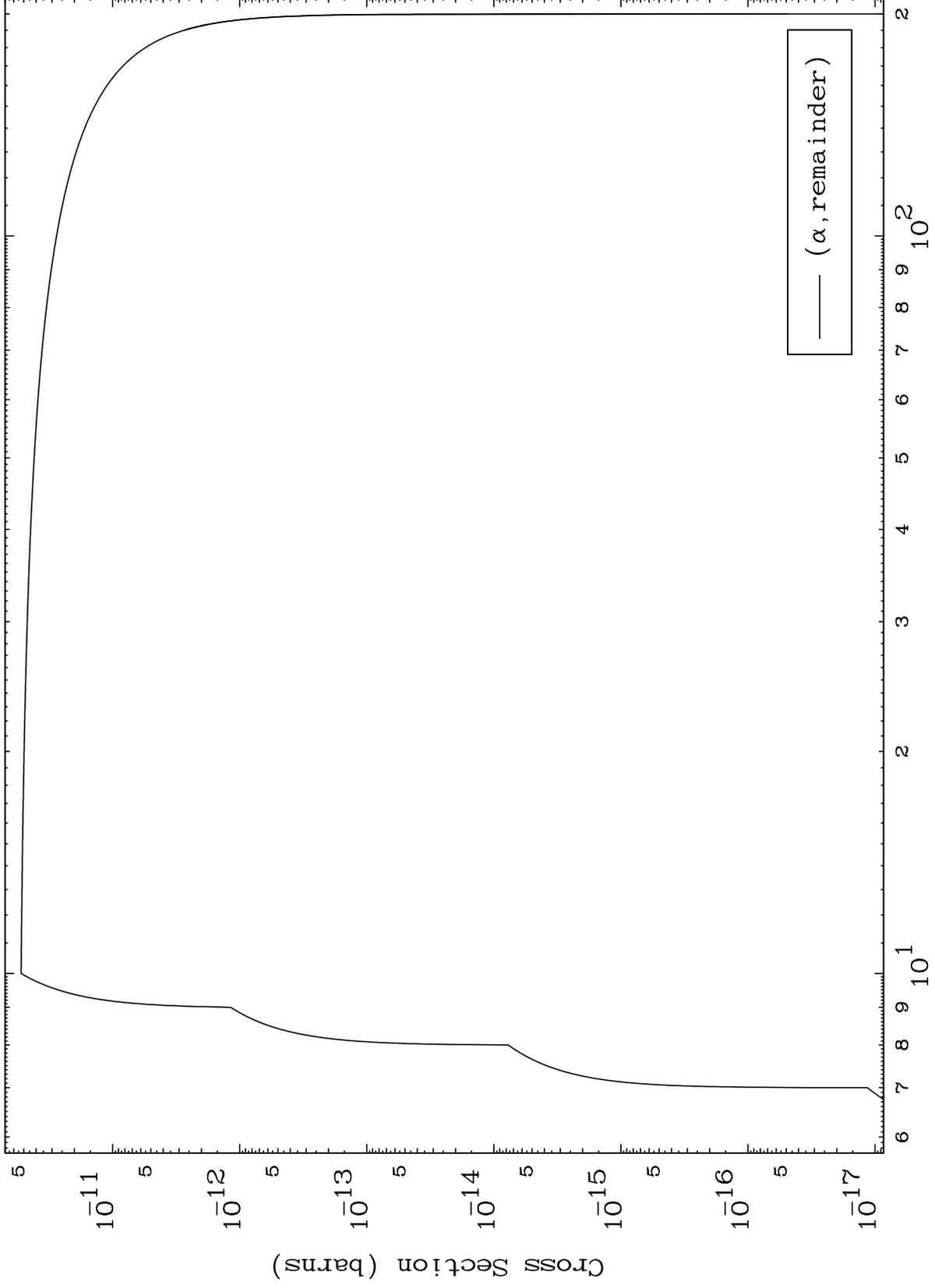
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Press Mouse Button to Start

MAT 9222

$\alpha$  Major  
0 Kelvin Cross Sections

92-U -233



1

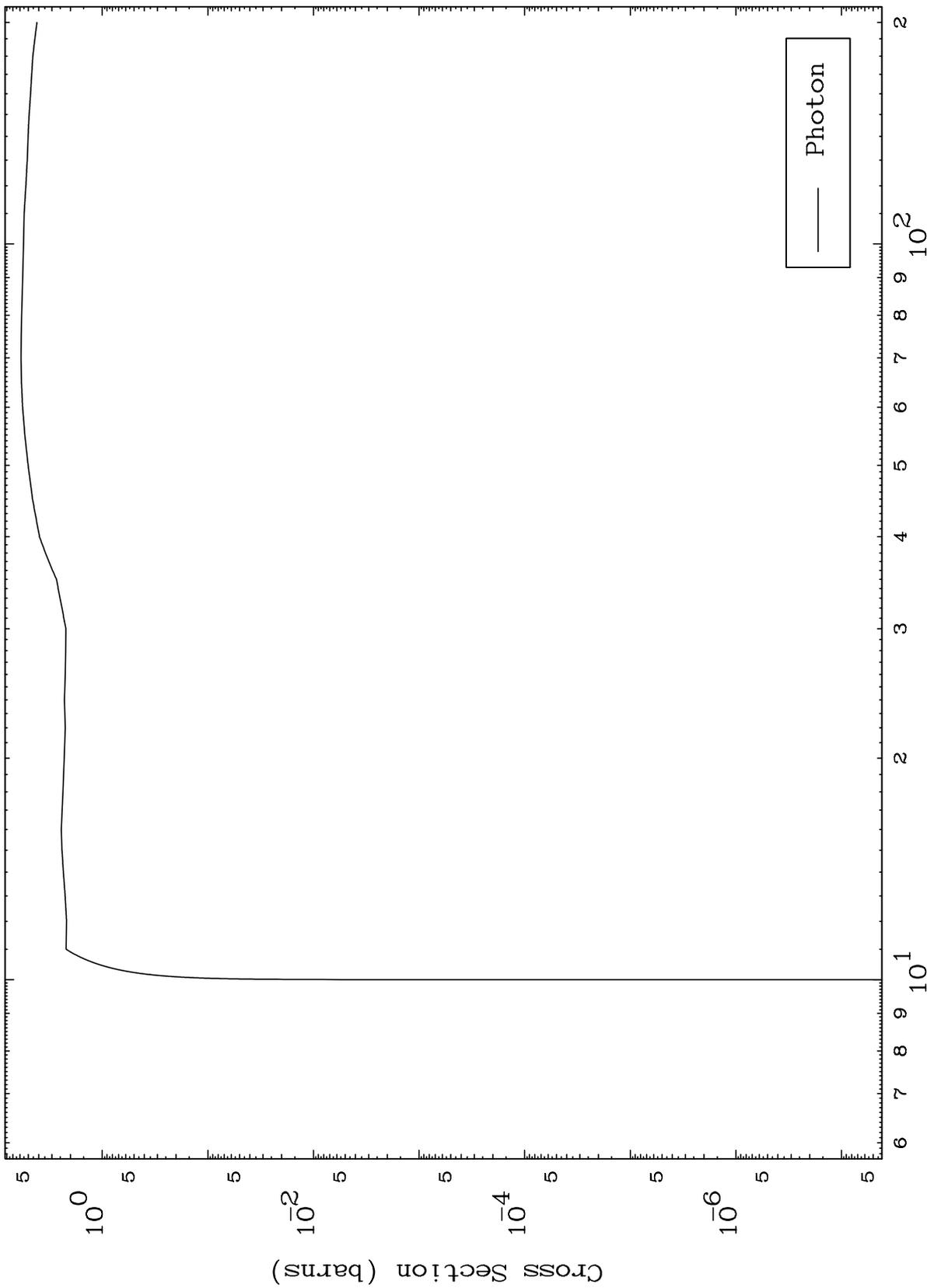
Incident Energy (MeV)

92-U -233

MAT 9222

92-U -233

$\alpha$  Fission  
Radionuclide Production Cross Section



92-U -233

Incident Energy (MeV)

2