

Program EVALPLOT  
(Version 2015-2)

by

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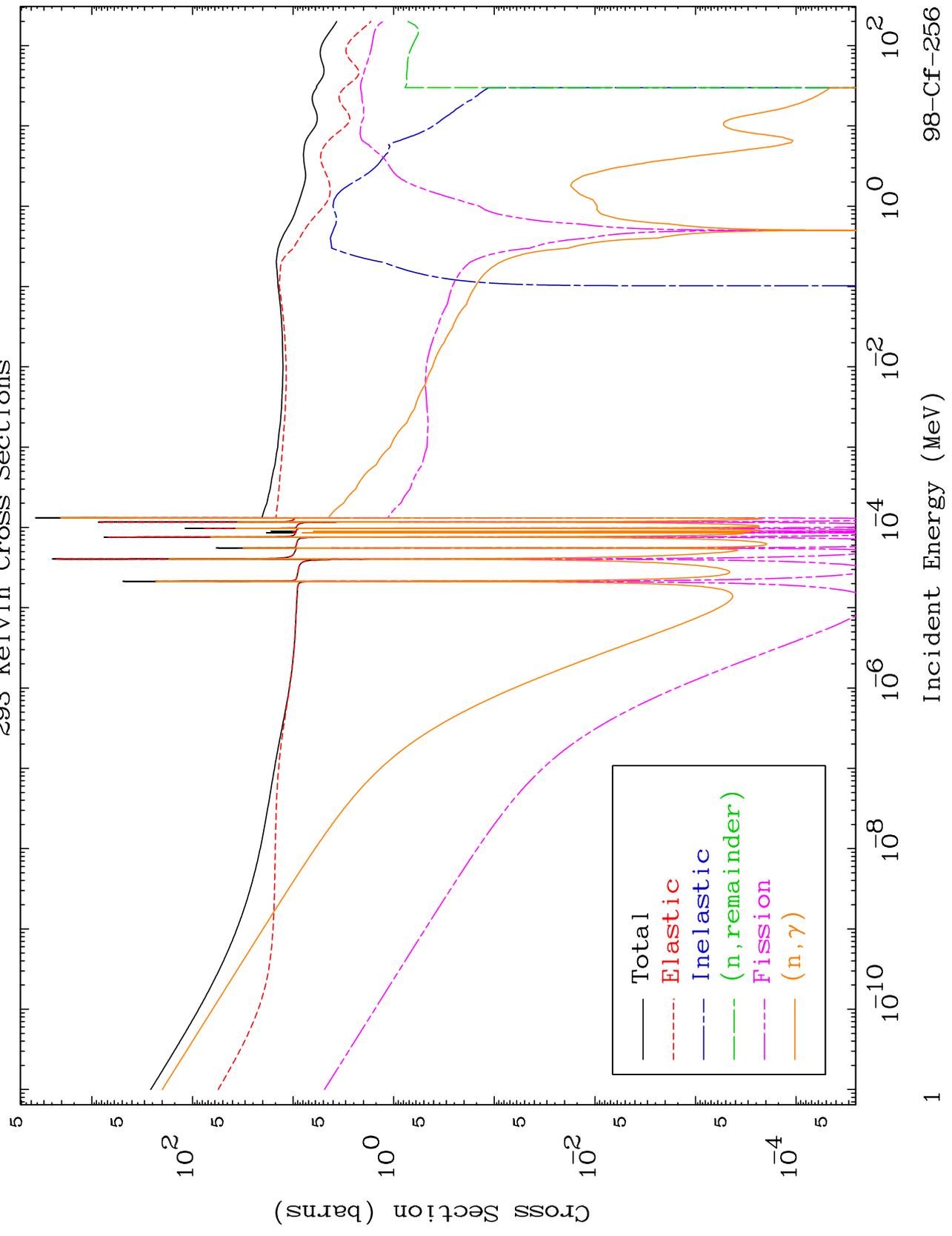
Press Mouse Button to Start

MAT 9873

Major

293 Kelvin Cross Sections

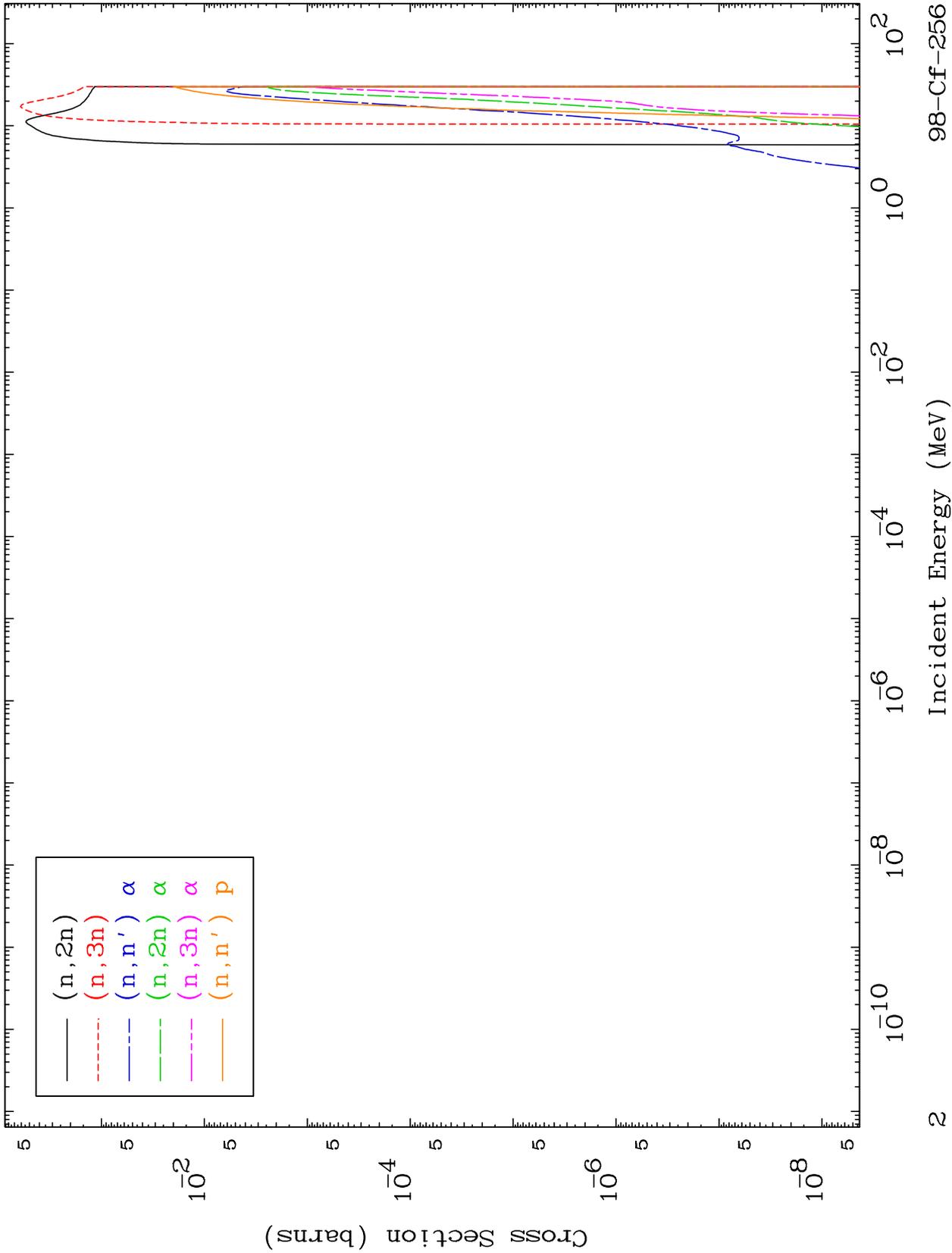
98-Cf-256



MAT 9873

Neutron Production  
293 Kelvin Cross Sections

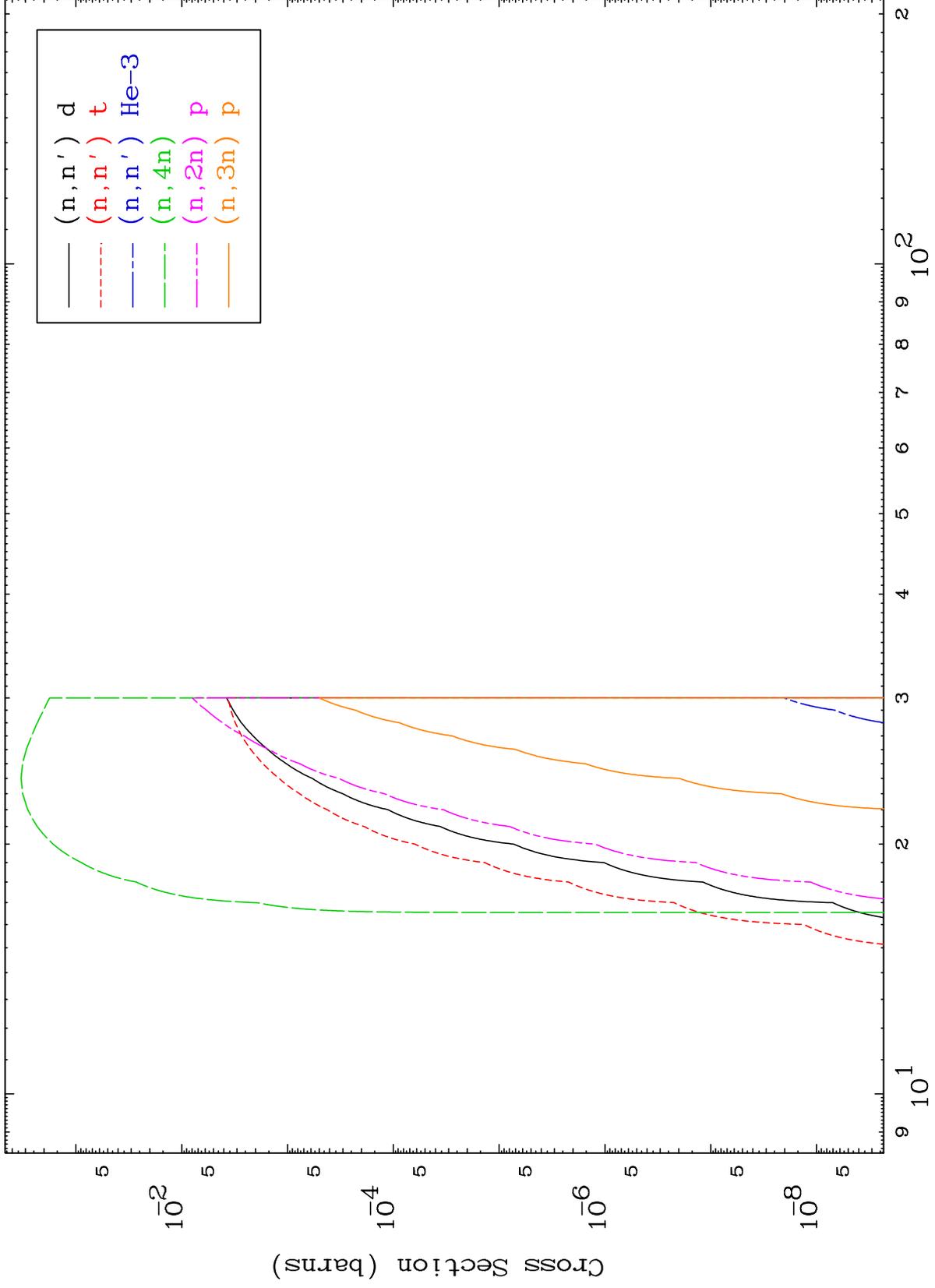
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Neutron Production  
293 Kelvin Cross Sections

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Incident Energy (MeV)

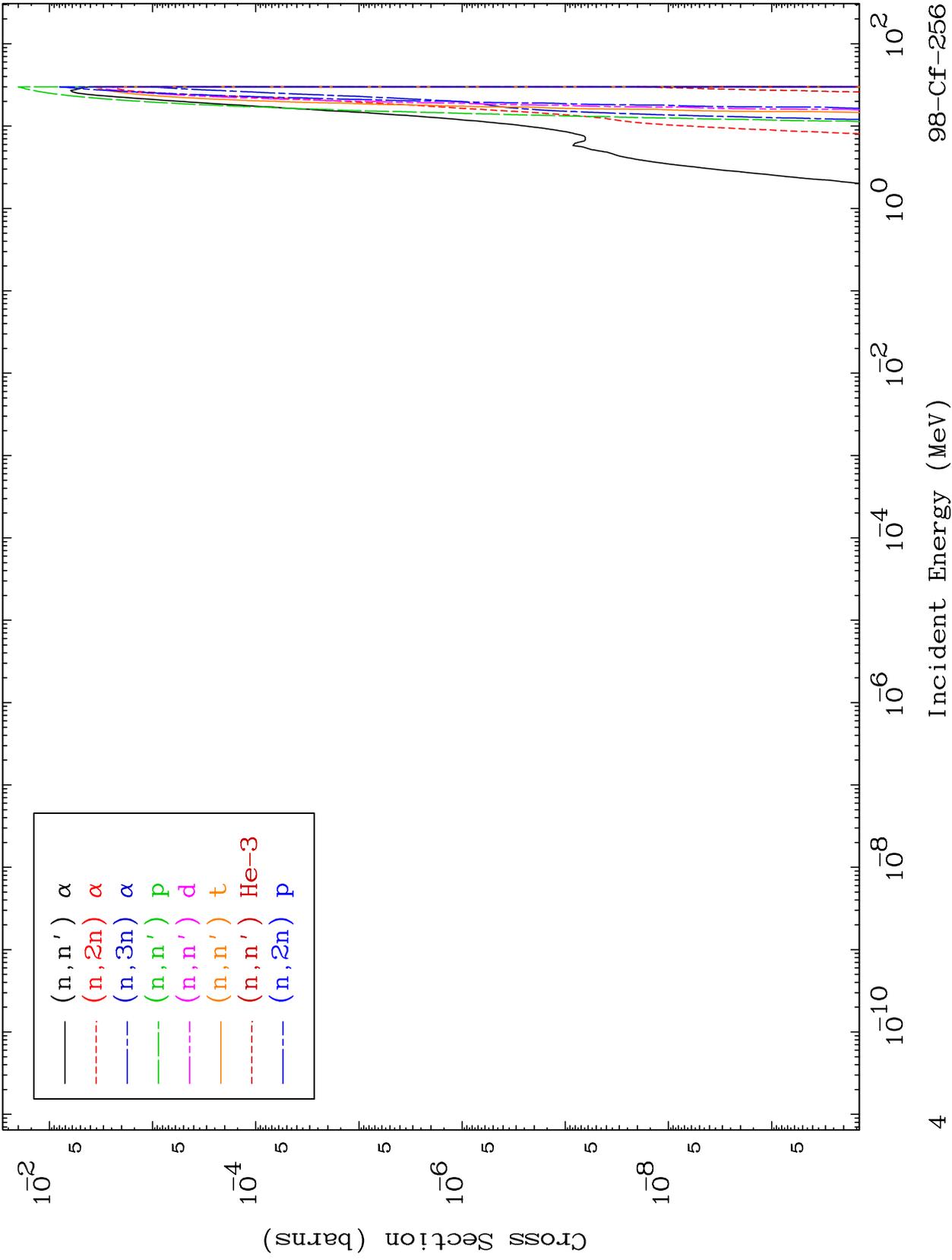
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3

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Charged Particle  
293 Kelvin Cross Sections

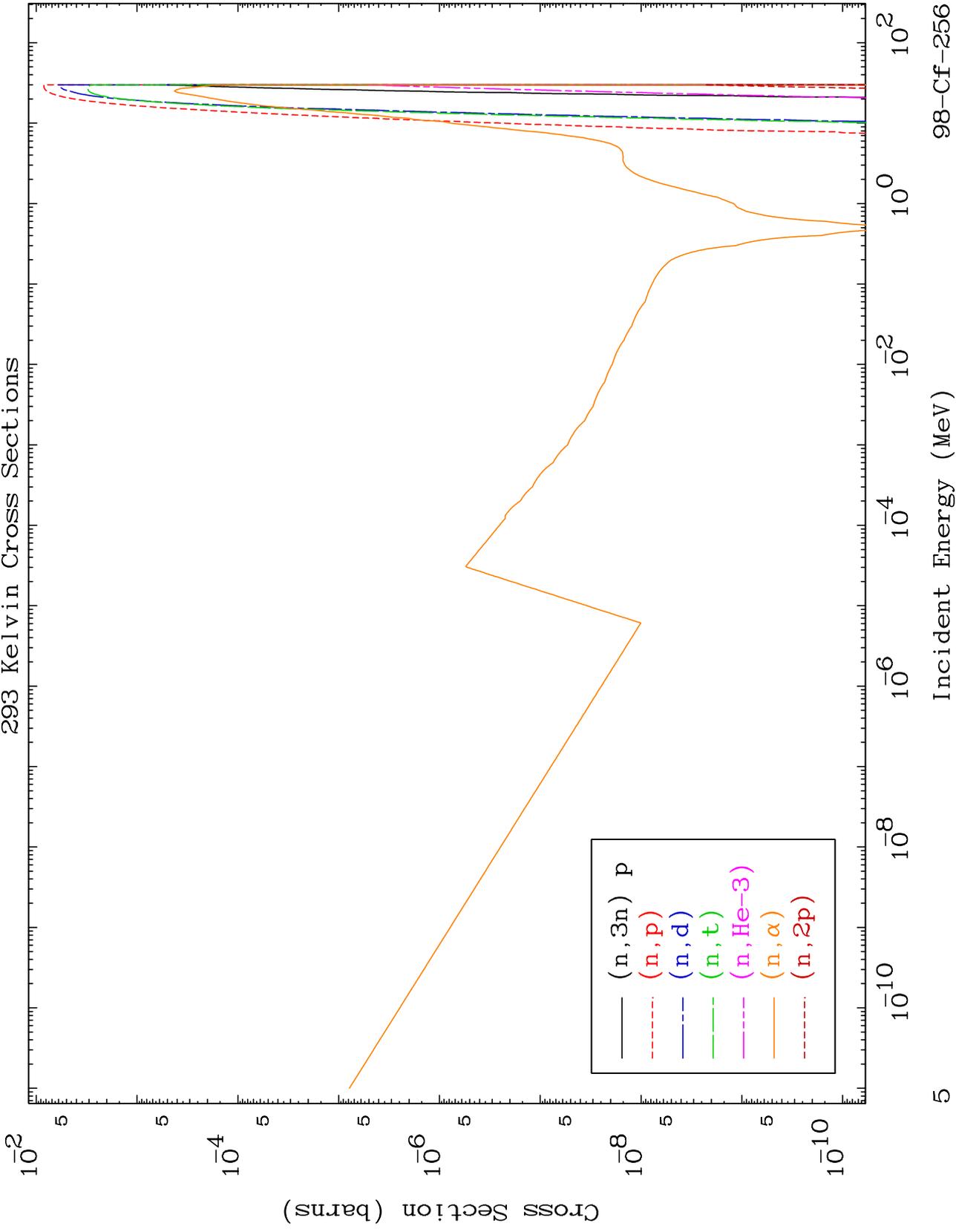
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Charged Particle  
293 Kelvin Cross Sections

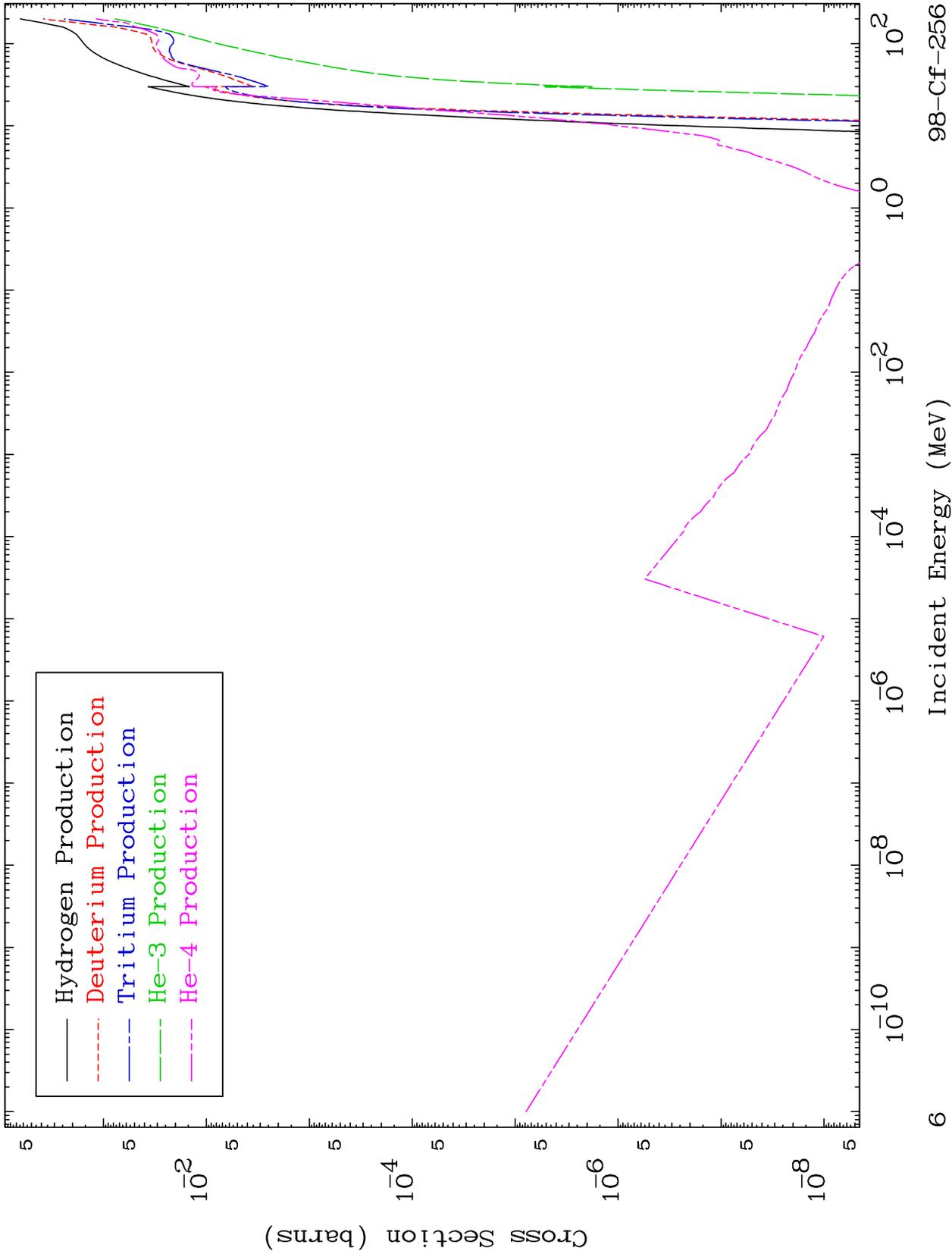
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Particle Production  
293 Kelvin Cross Sections

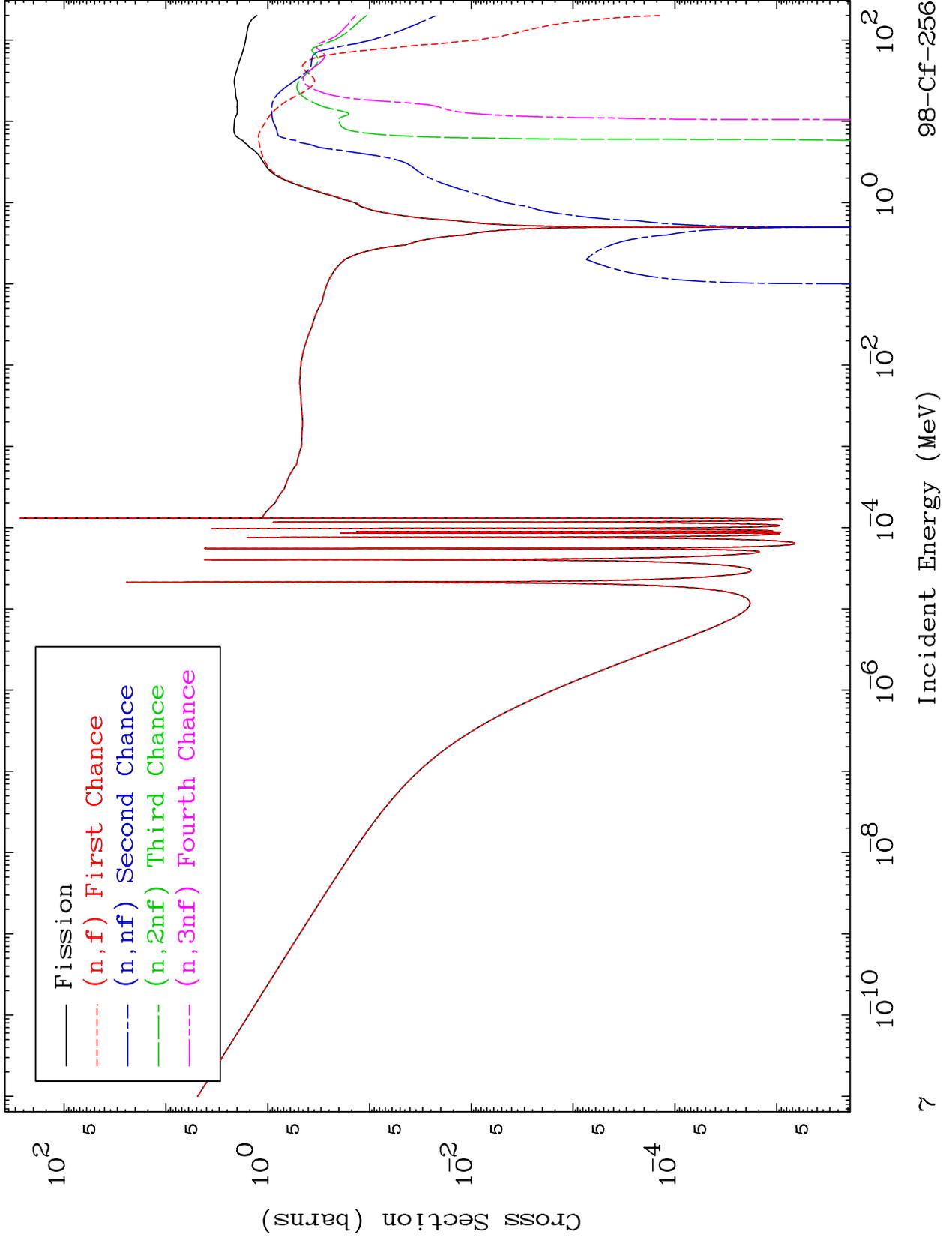
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Fission  
293 Kelvin Cross Sections

98-Cf-256

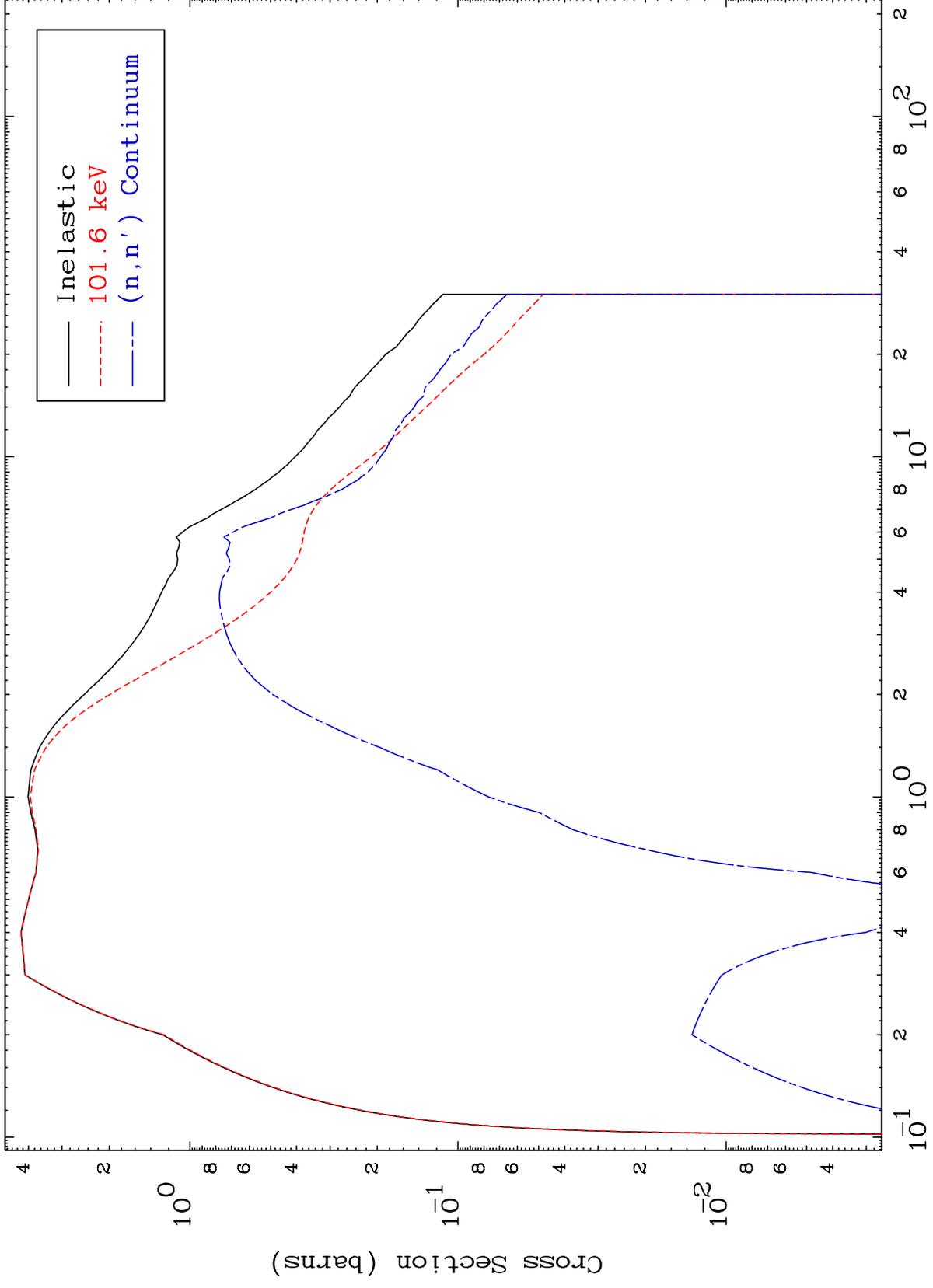


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(n,n') Level

98-Cf-256

293 Kelvin Cross Sections



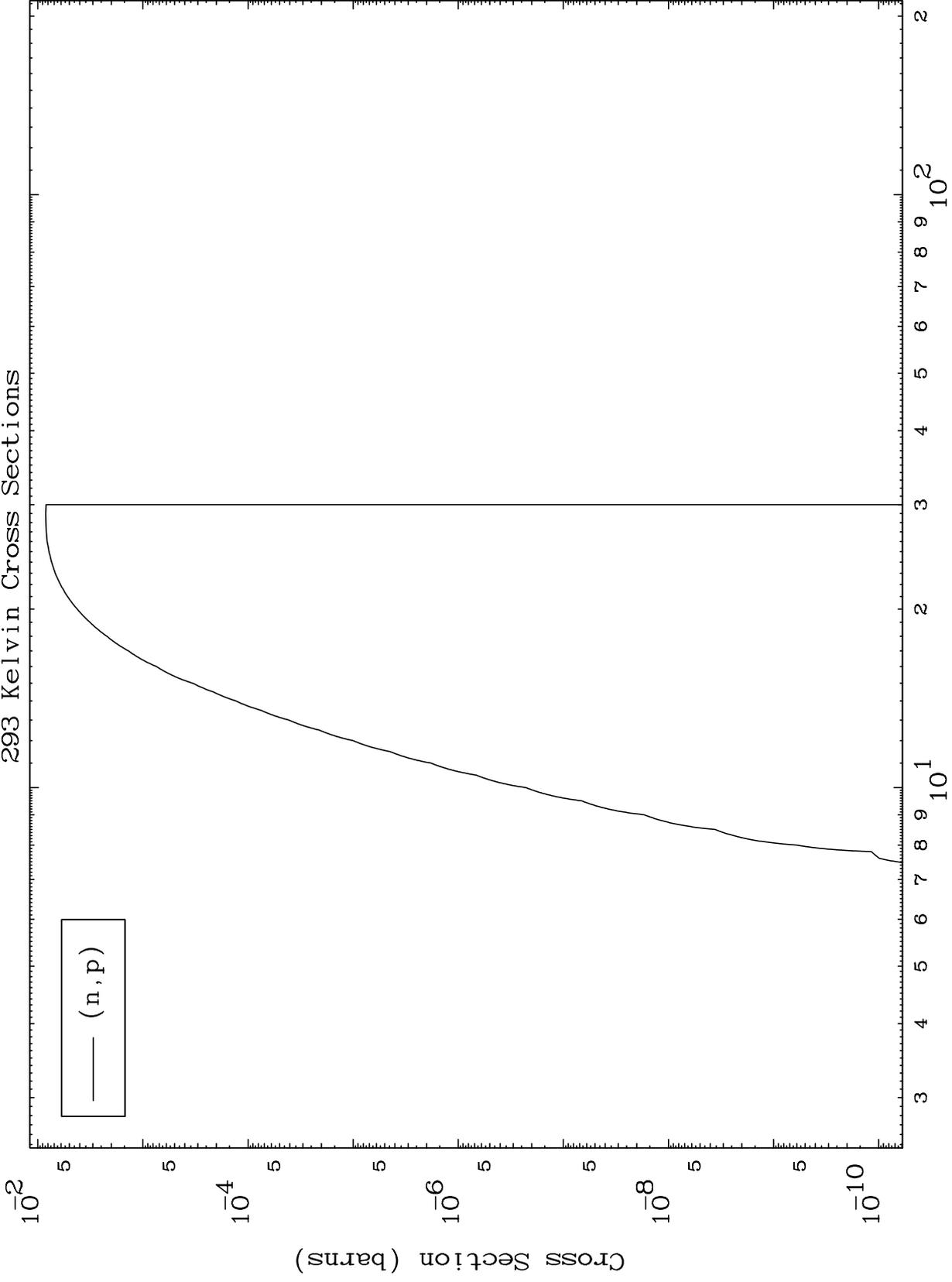
Incident Energy (MeV)

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(n,p) Levels  
293 Kelvin Cross Sections

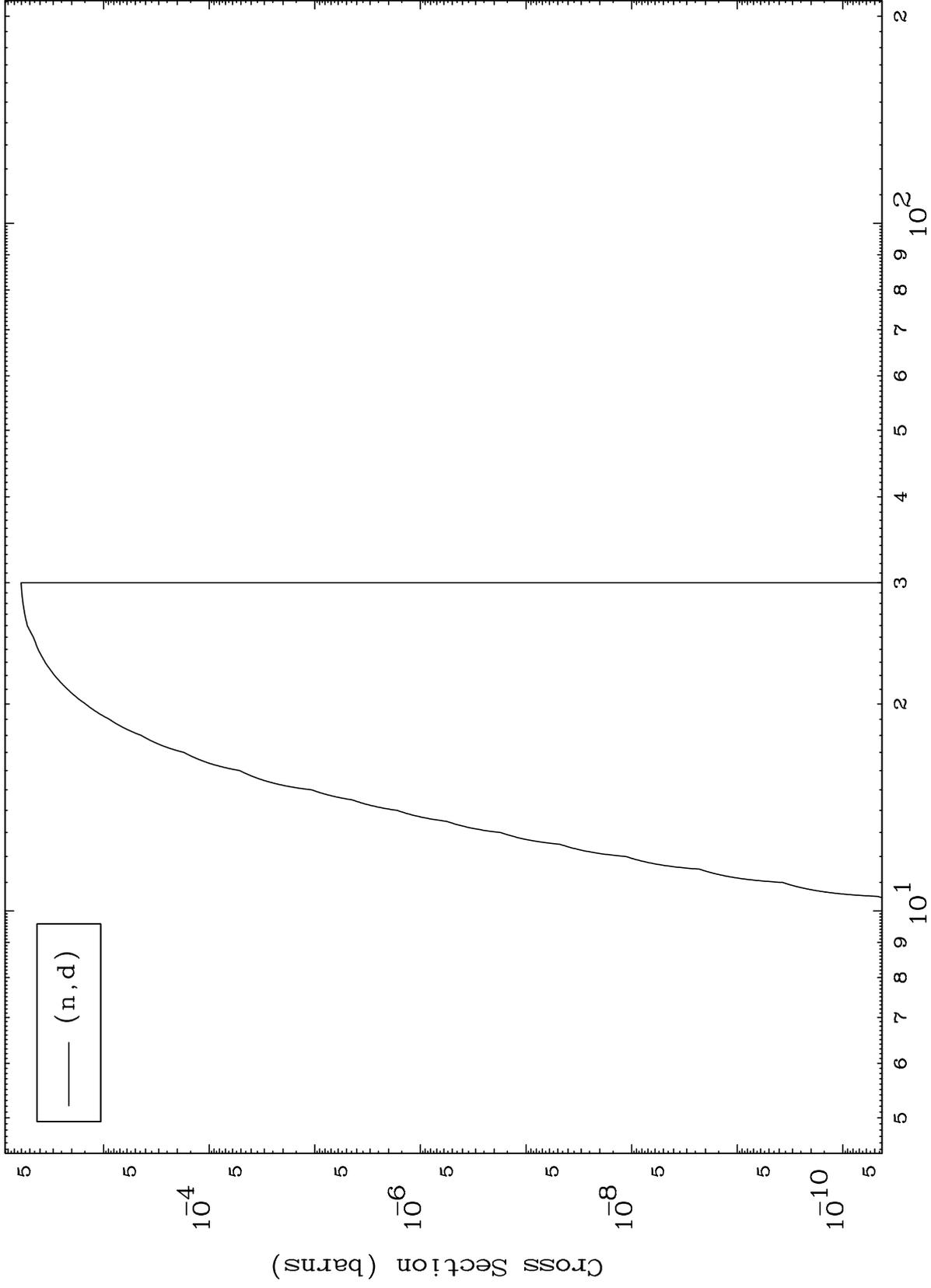
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(n,d) Levels  
293 Kelvin Cross Sections

98-Cf-256



10

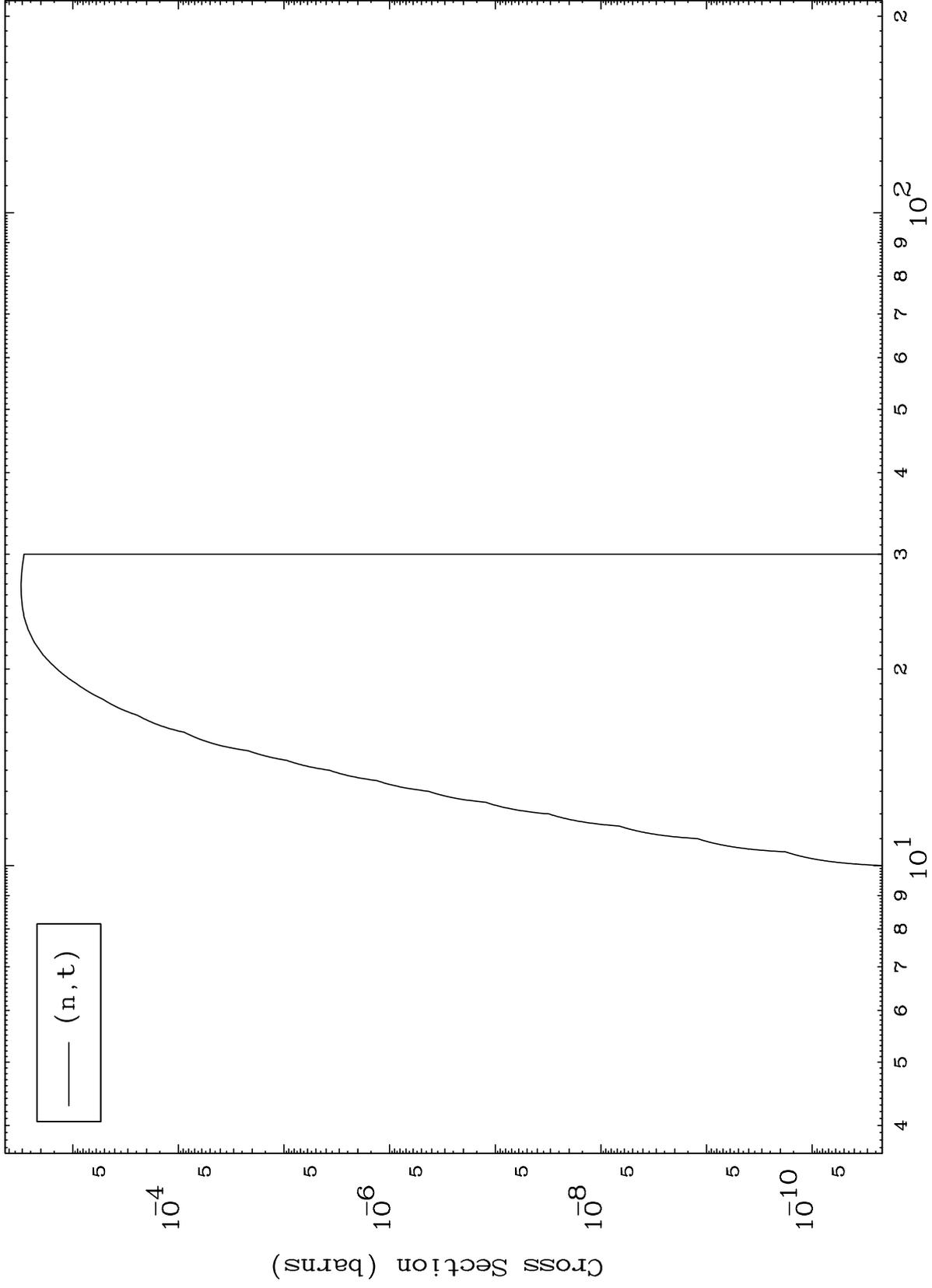
Incident Energy (MeV)

98-Cf-256

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(n,t) Levels  
293 Kelvin Cross Sections

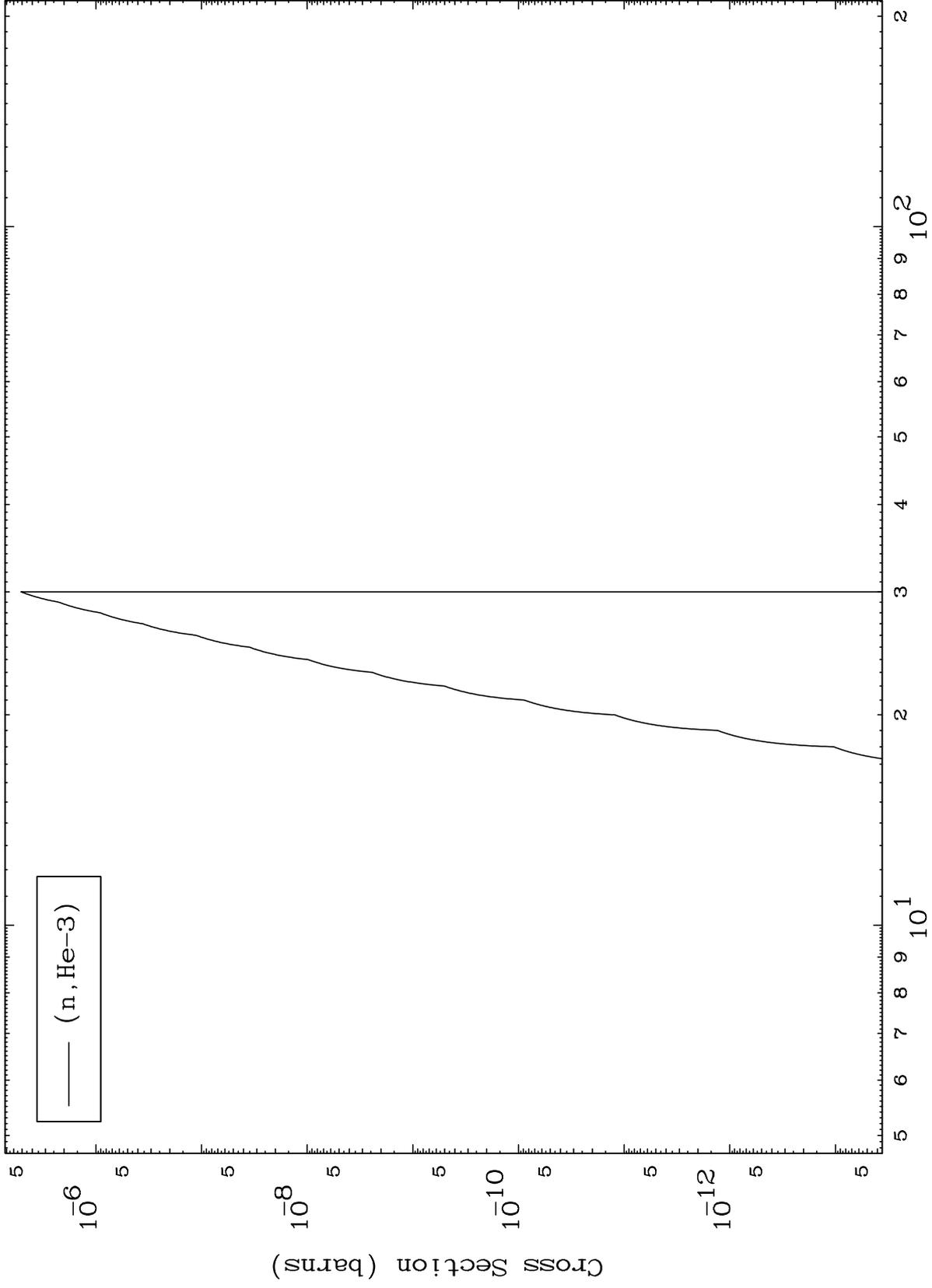
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(n,He3) Levels  
293 Kelvin Cross Sections

98-Cf-256



12

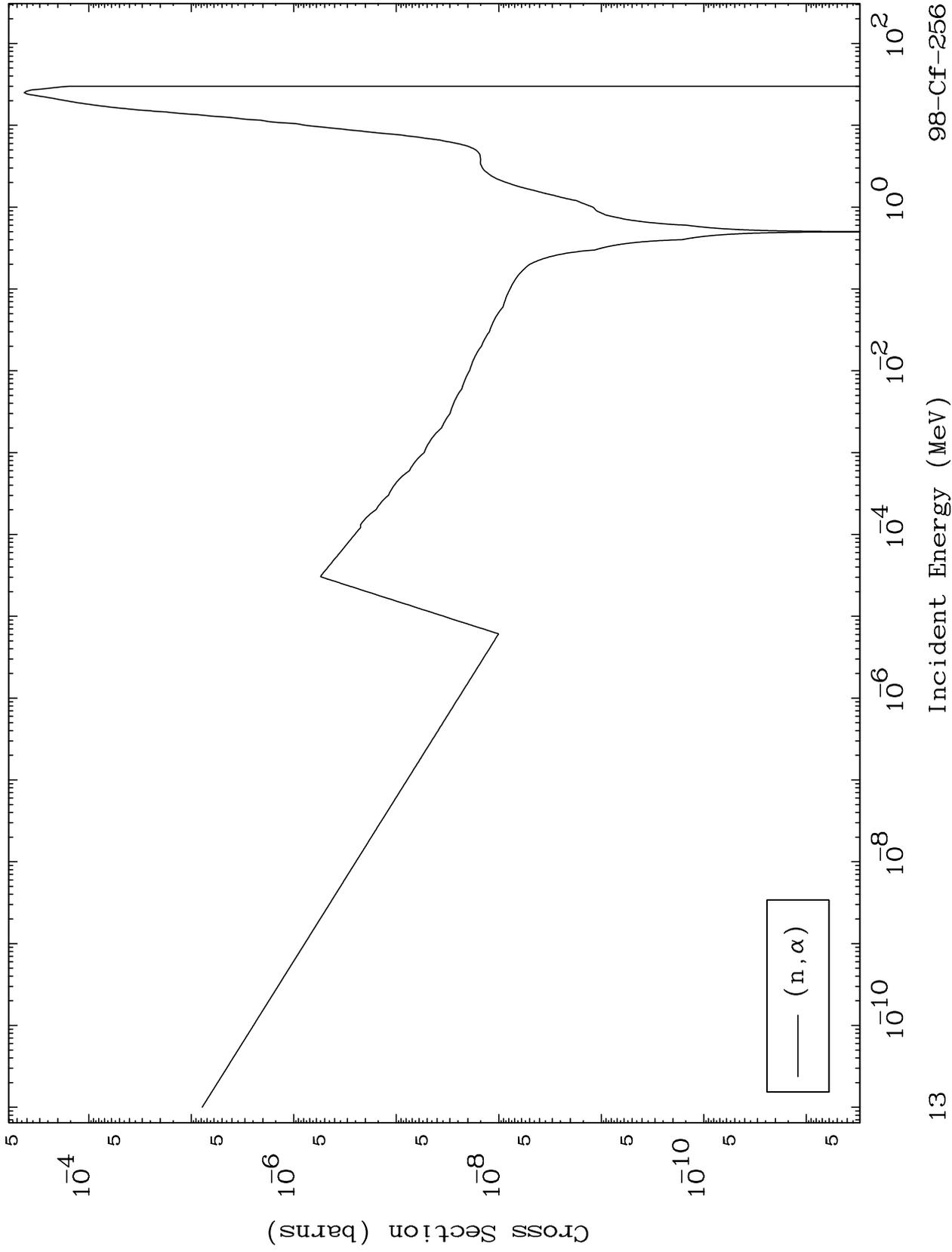
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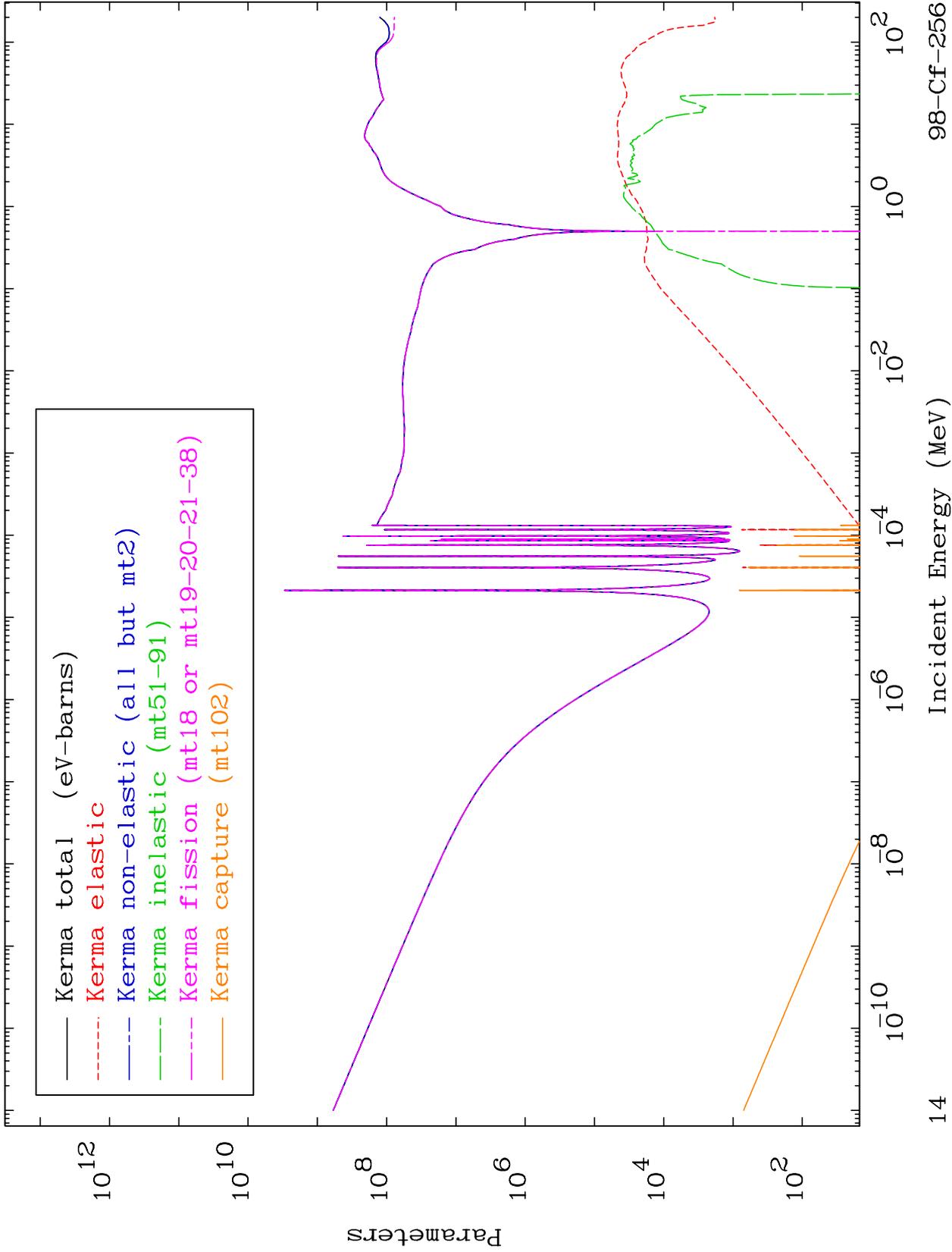
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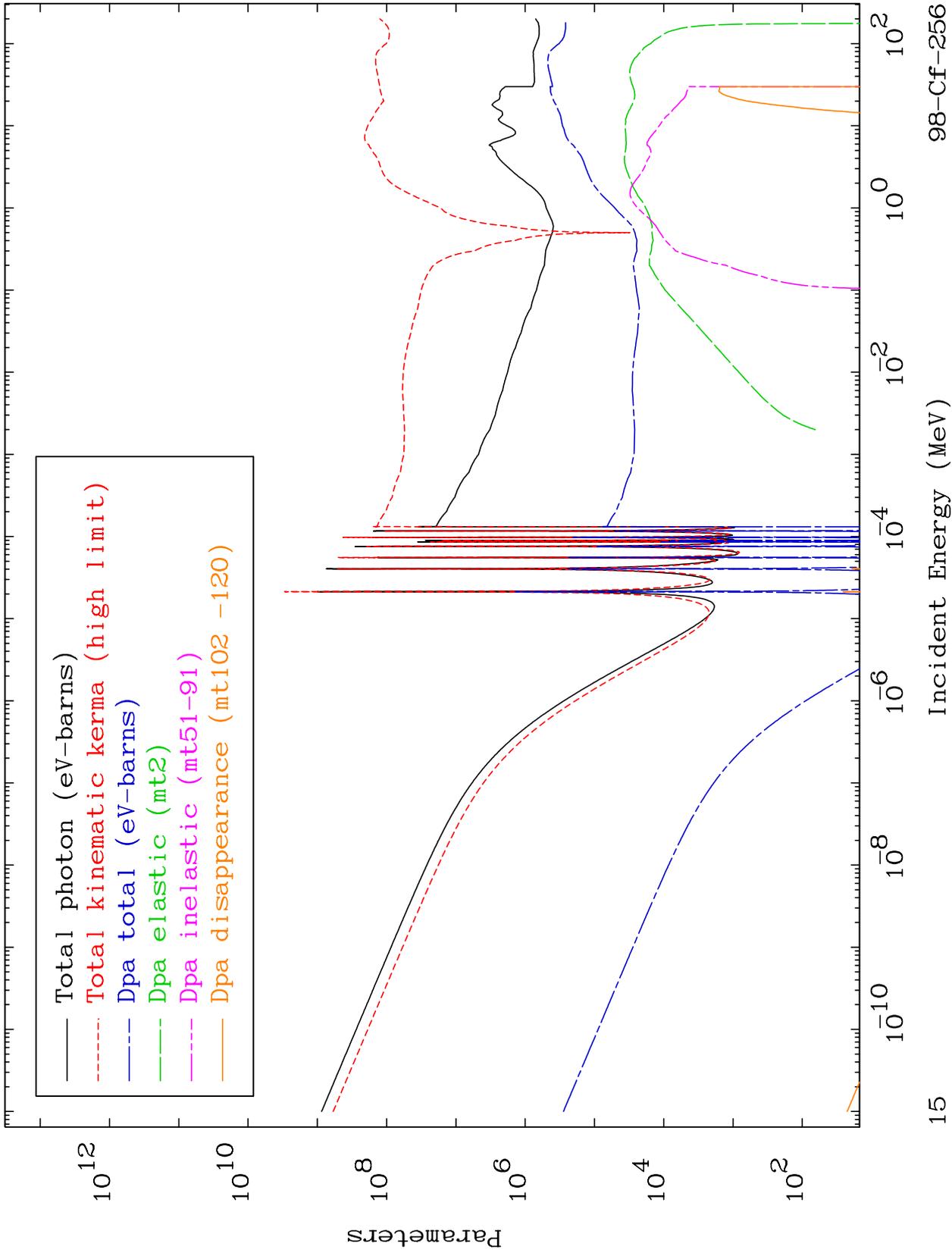
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

98-Cf-256



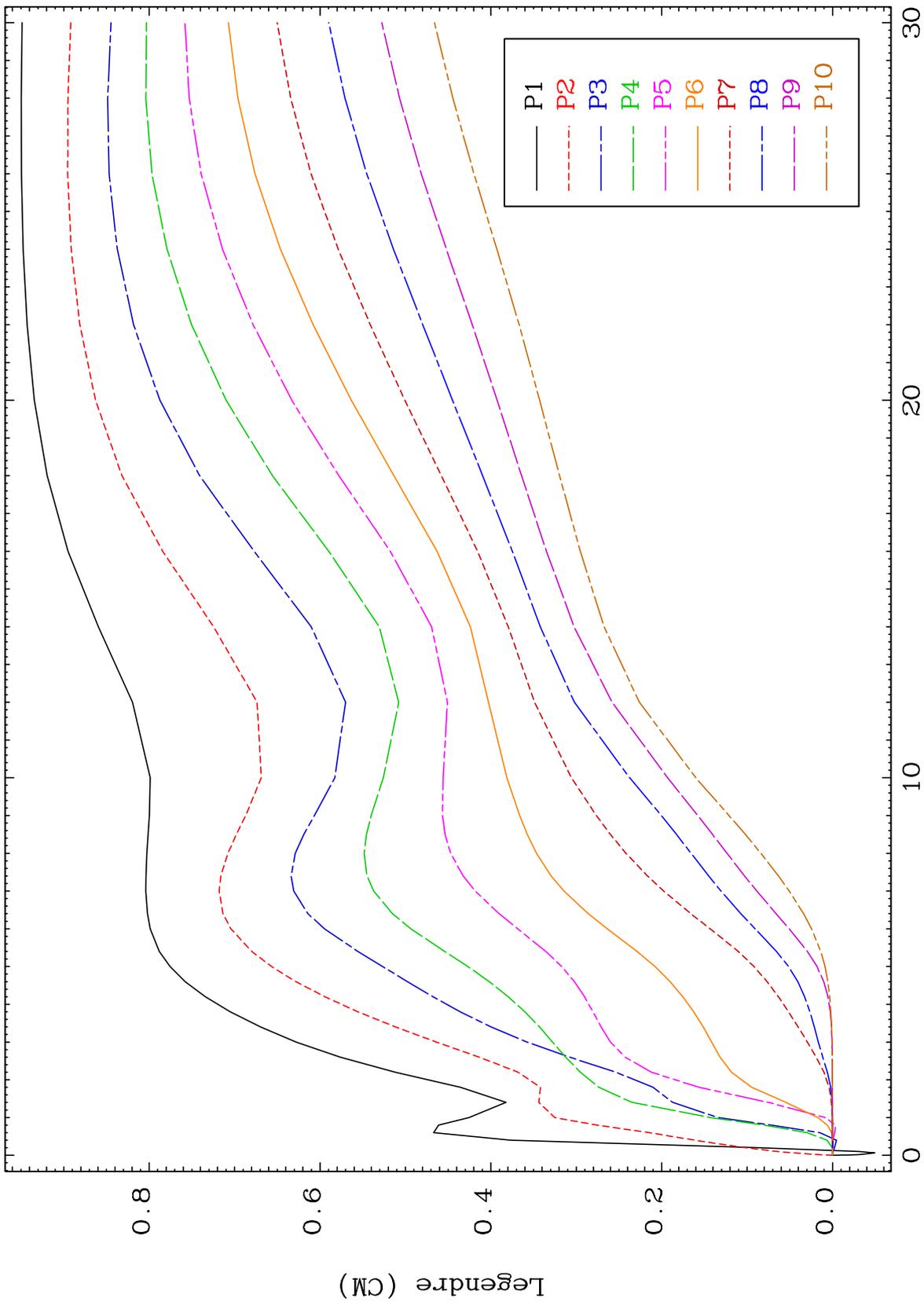




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Elastic Legendre Coefficients

98-Cf-256



16

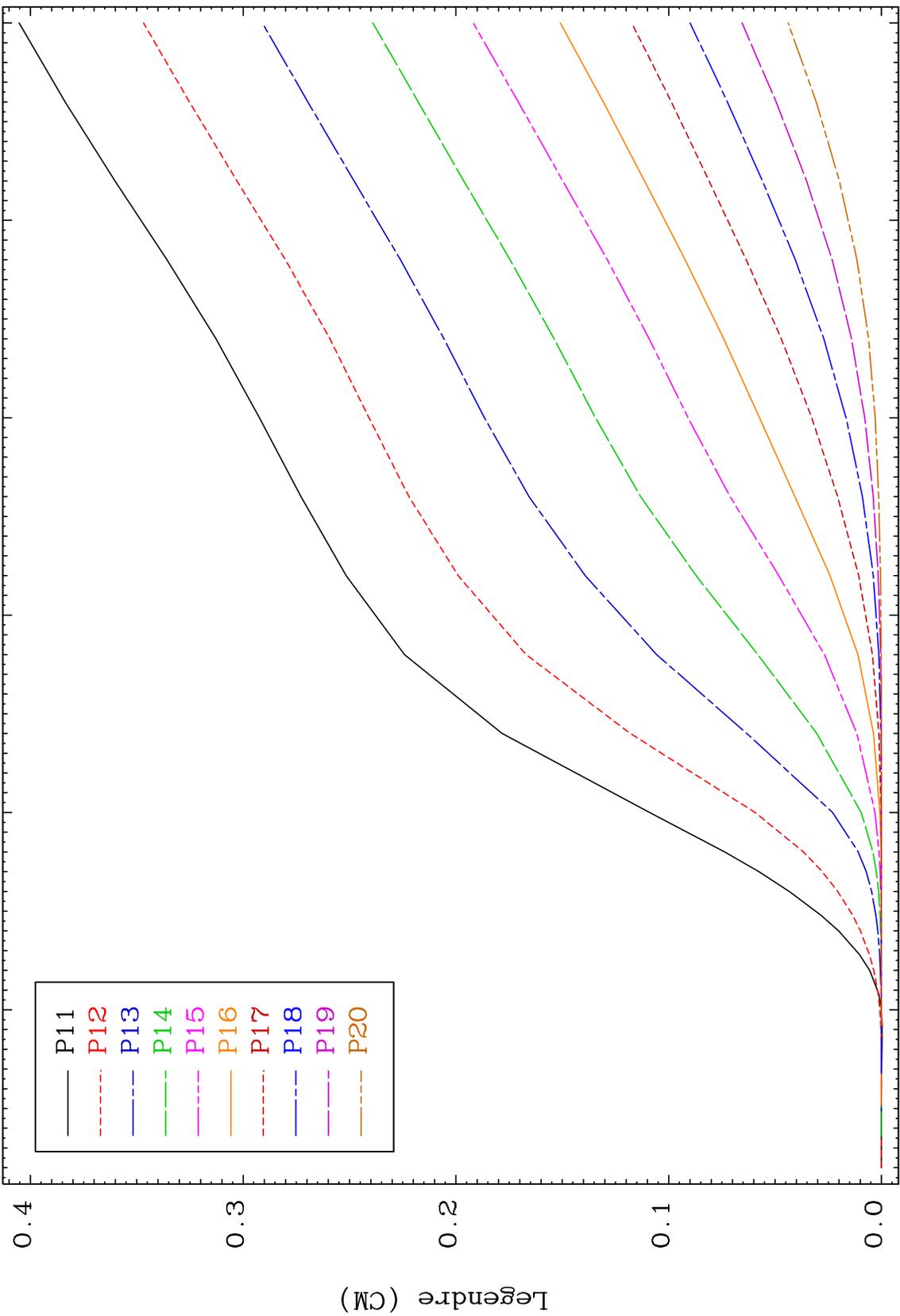
Incident Energy (MeV)

98-Cf-256

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Elastic Legendre Coefficients

98-Cf-256



17

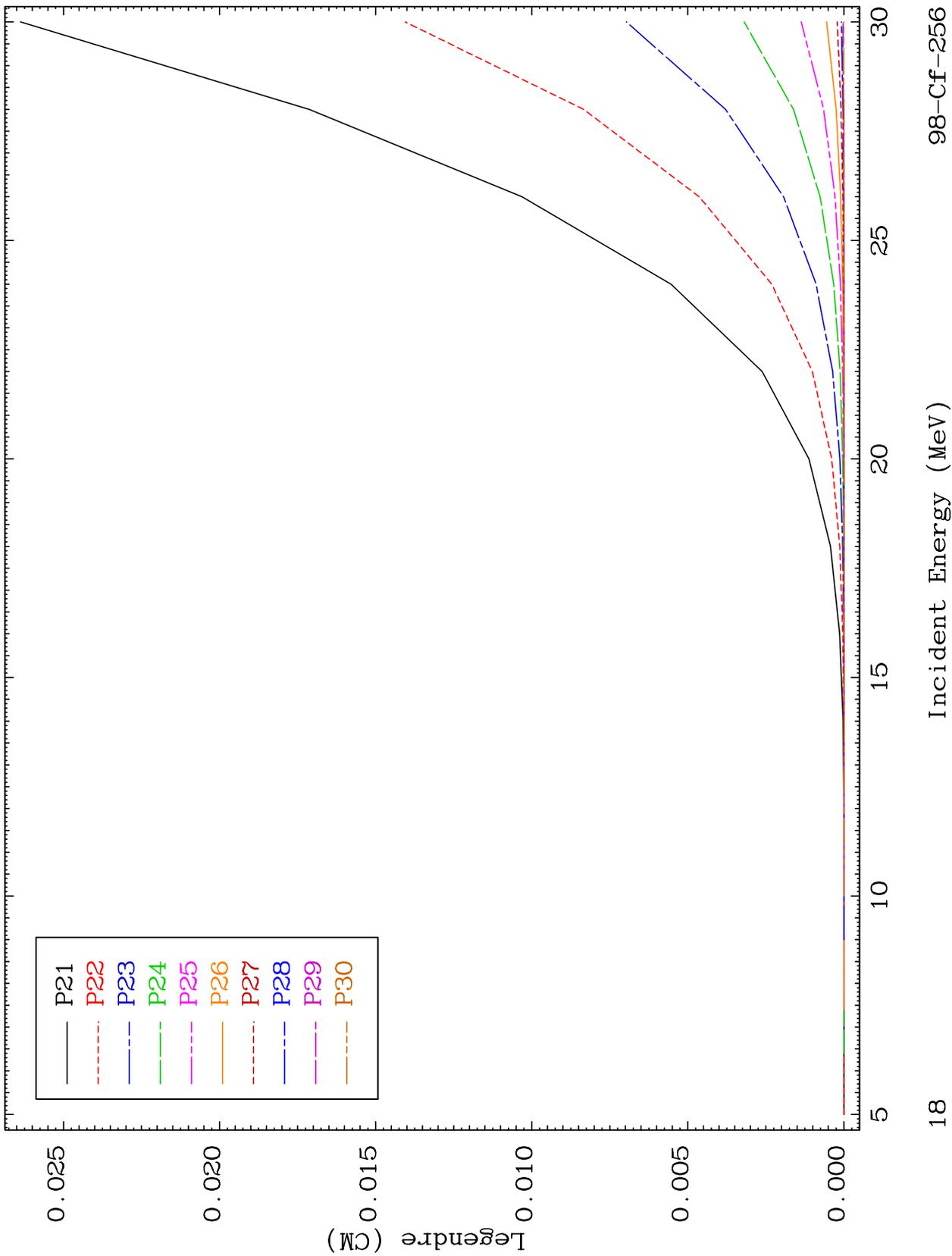
Incident Energy (MeV)

98-Cf-256

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### Elastic Legendre Coefficients

98-Cf-256



18

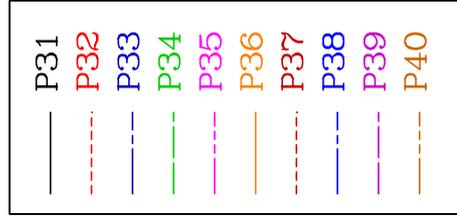
Incident Energy (MeV)

98-Cf-256

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Elastic  
Legendre Coefficients

98-Cf-256



$\times 10^{-6}$

3

Legendre (CM)

2

1

0

15

20

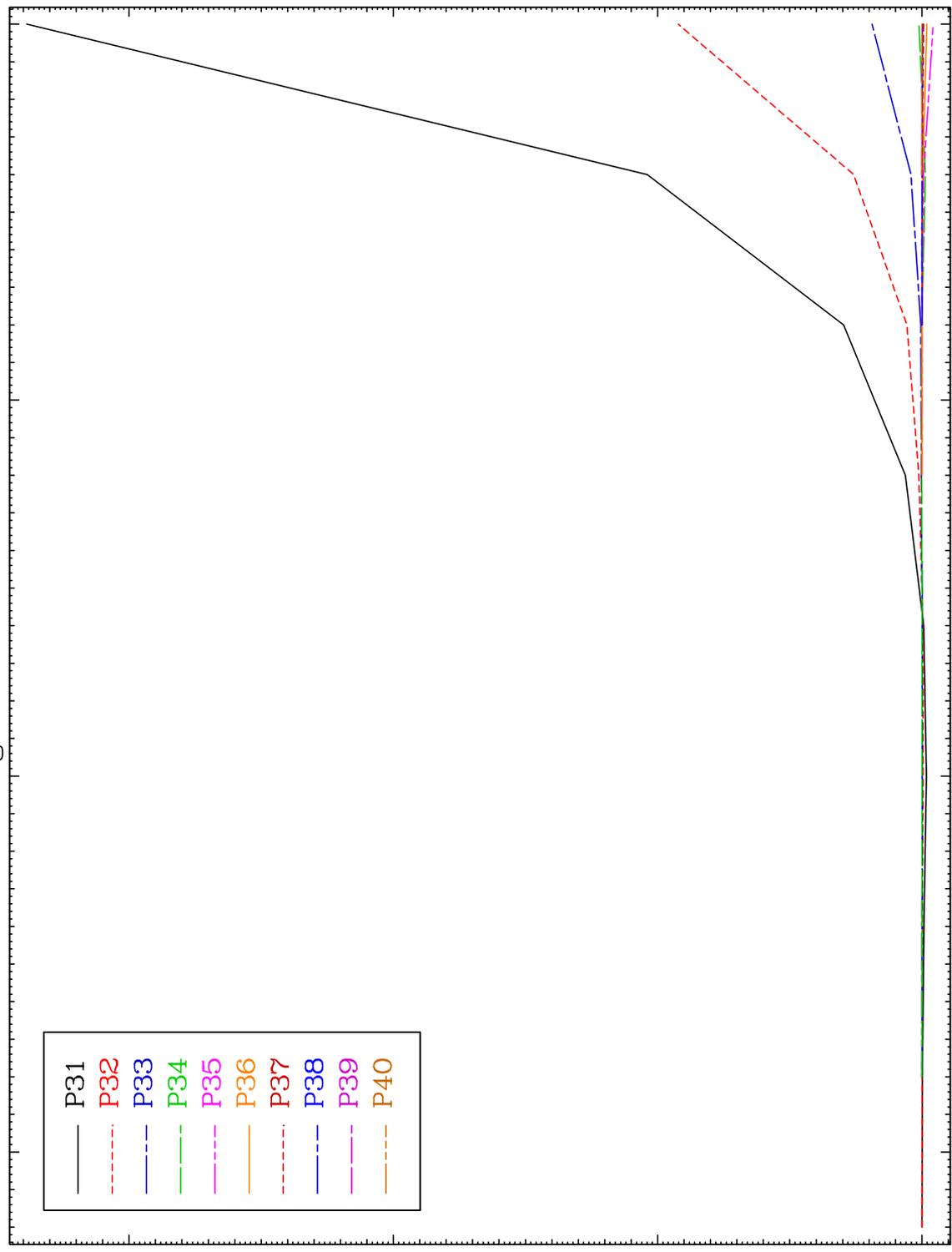
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30

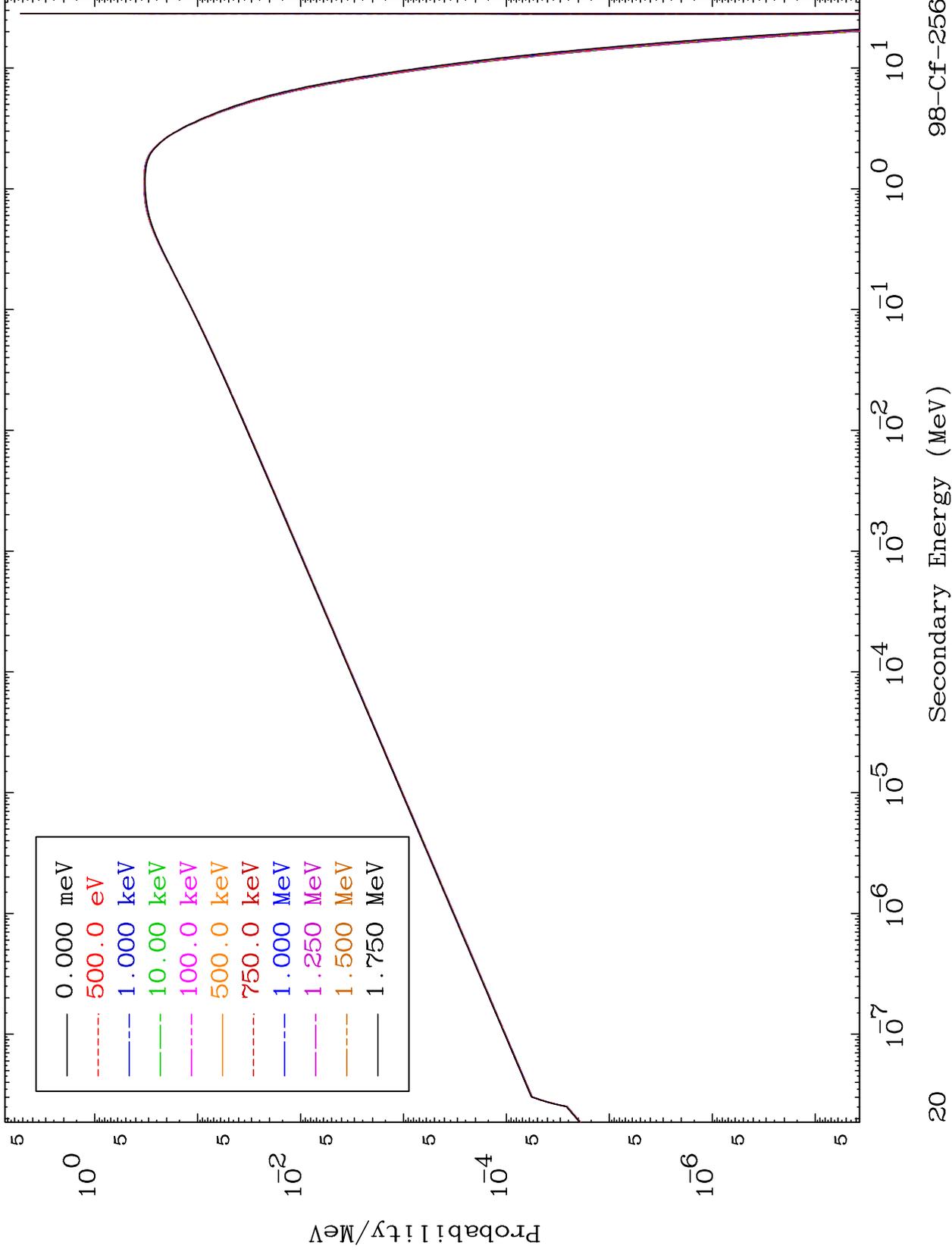
19

Incident Energy (MeV)

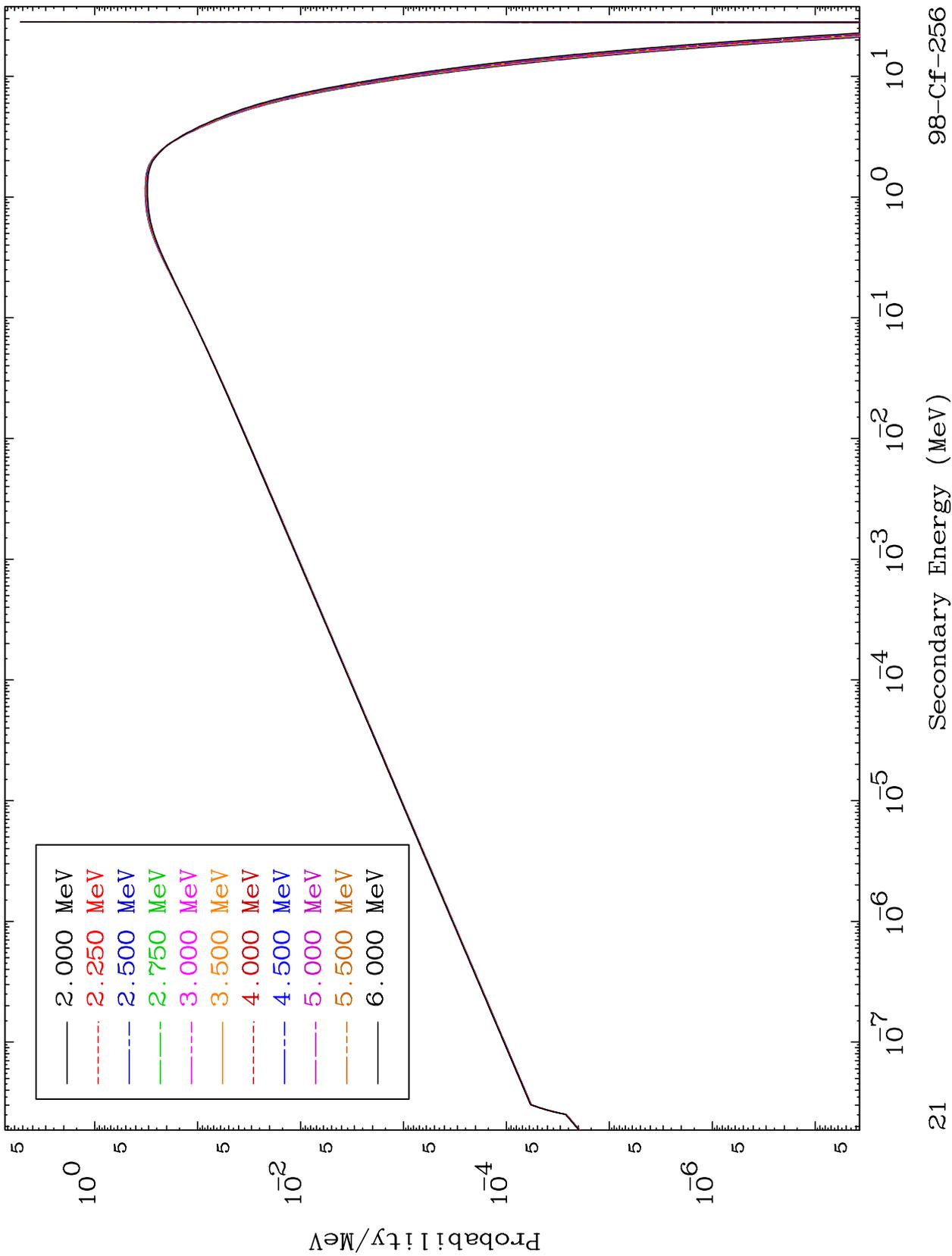
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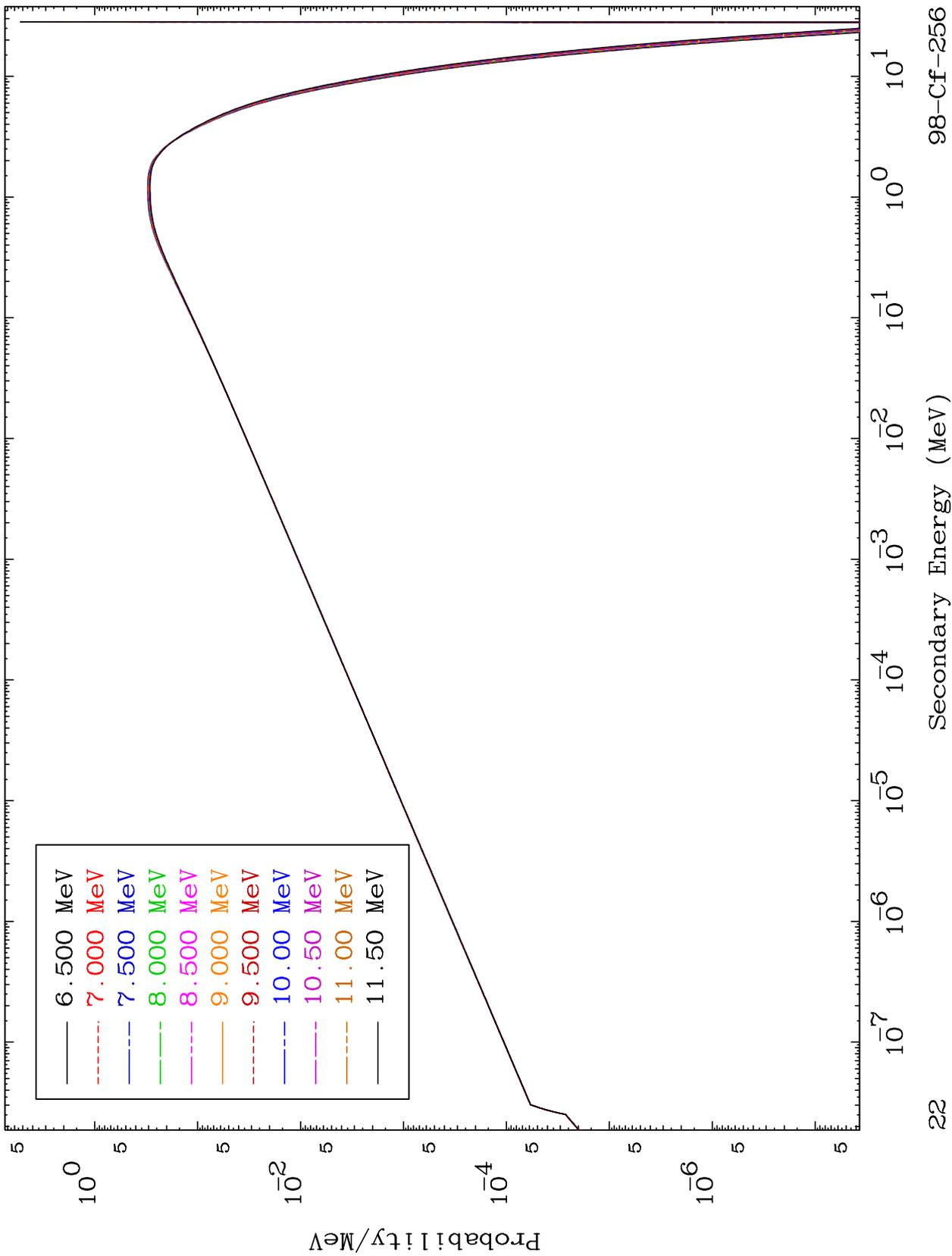
Fission  
Energy Distribution

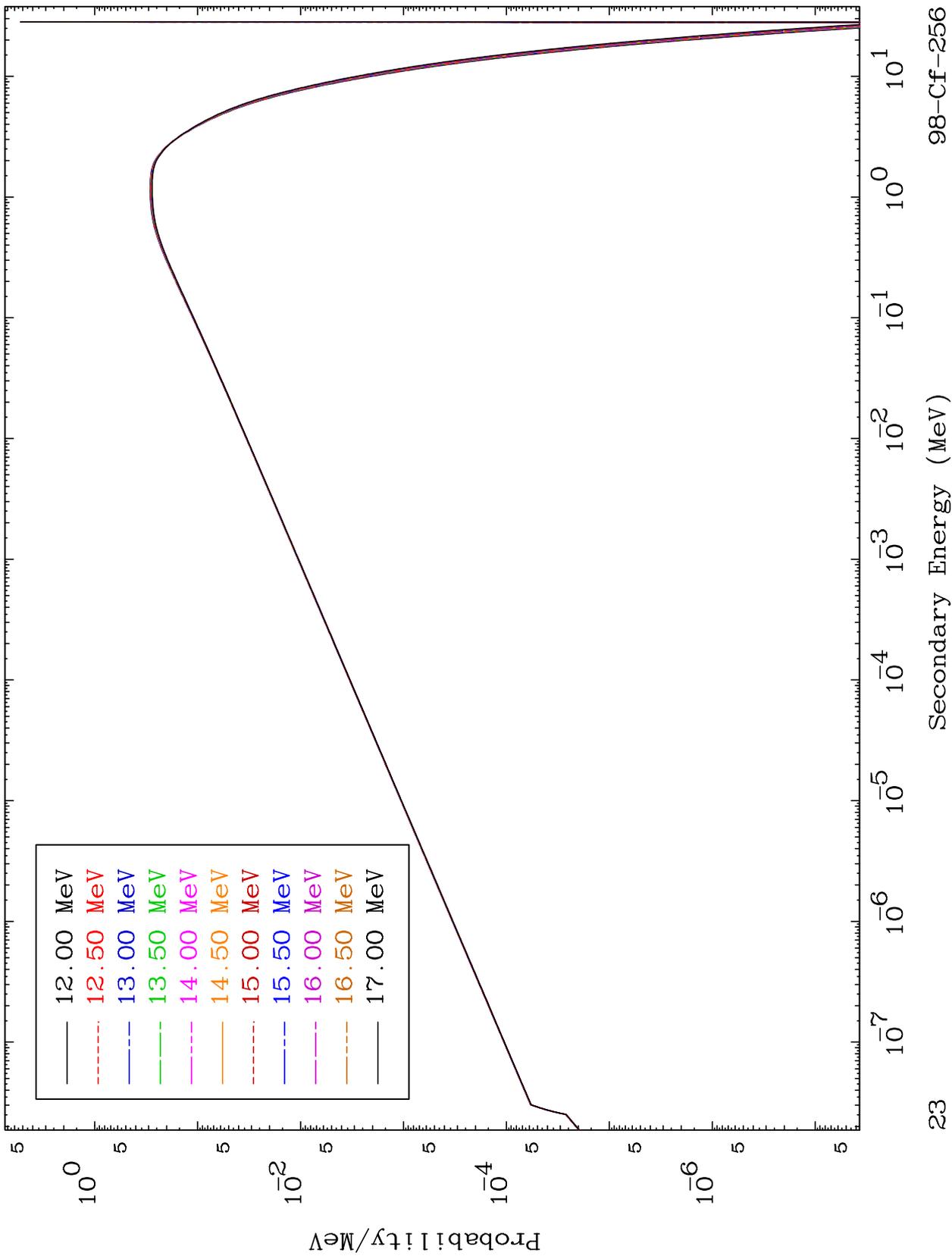


Fission  
Energy Distribution



Fission  
Energy Distribution

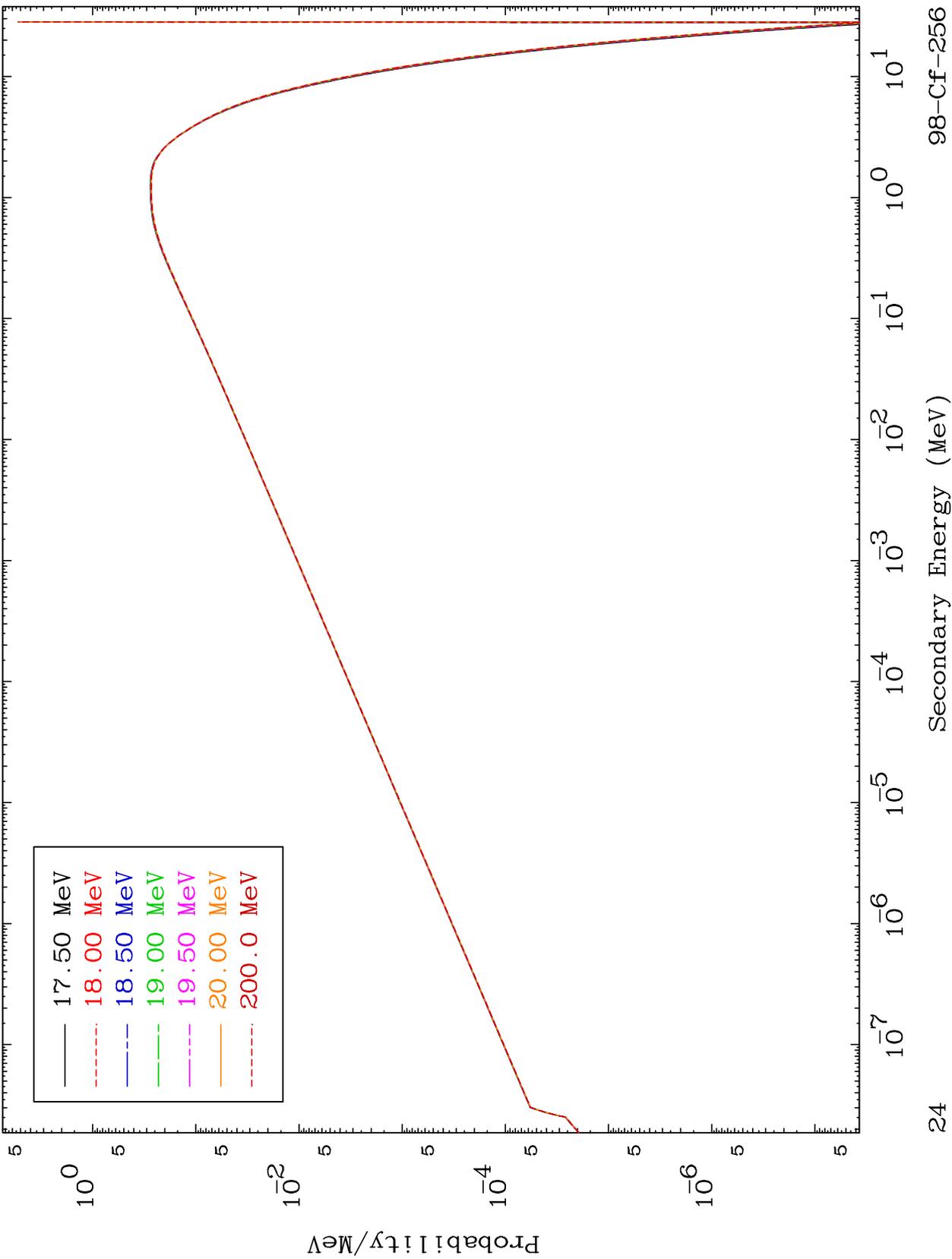




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### Fission Energy Distribution

98-Cf-256



98-Cf-256

Secondary Energy (MeV)