

Program EVALPLOT
(Version 2015-2)

by

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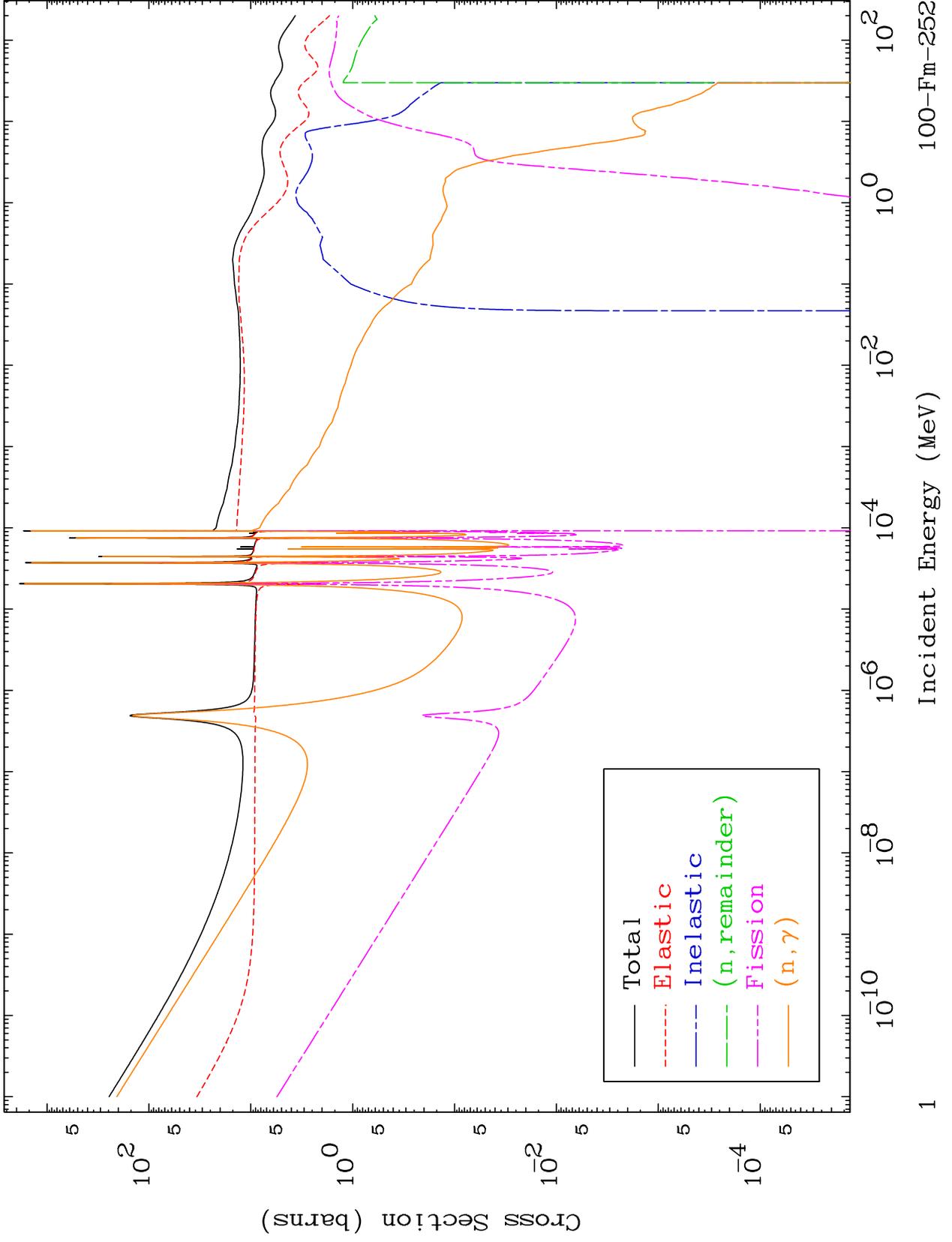
Press Mouse Button to Start

MAT 9933

Major

293 Kelvin Cross Sections

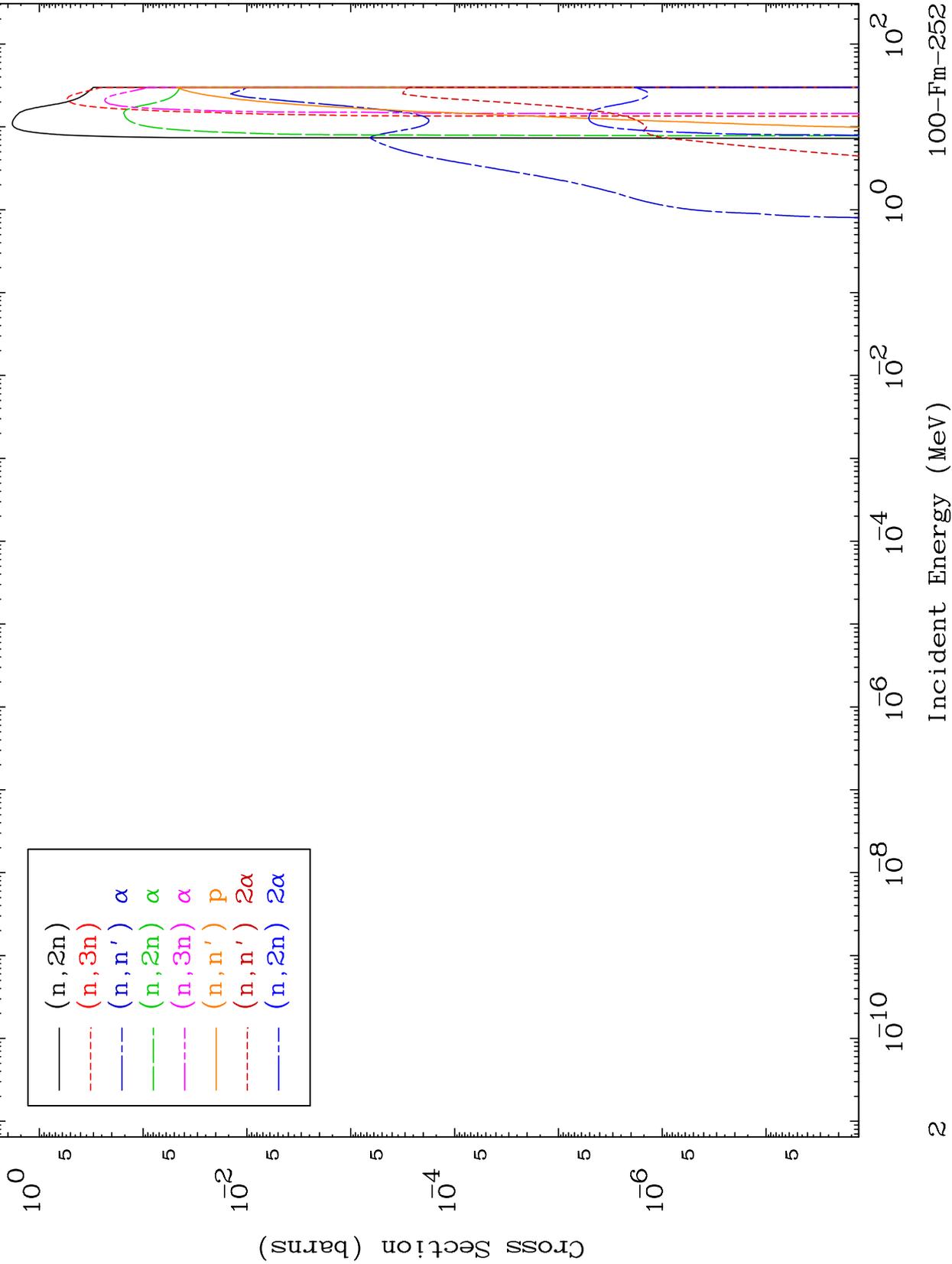
100-Fm-252



MAT 9933

Neutron Production
293 Kelvin Cross Sections

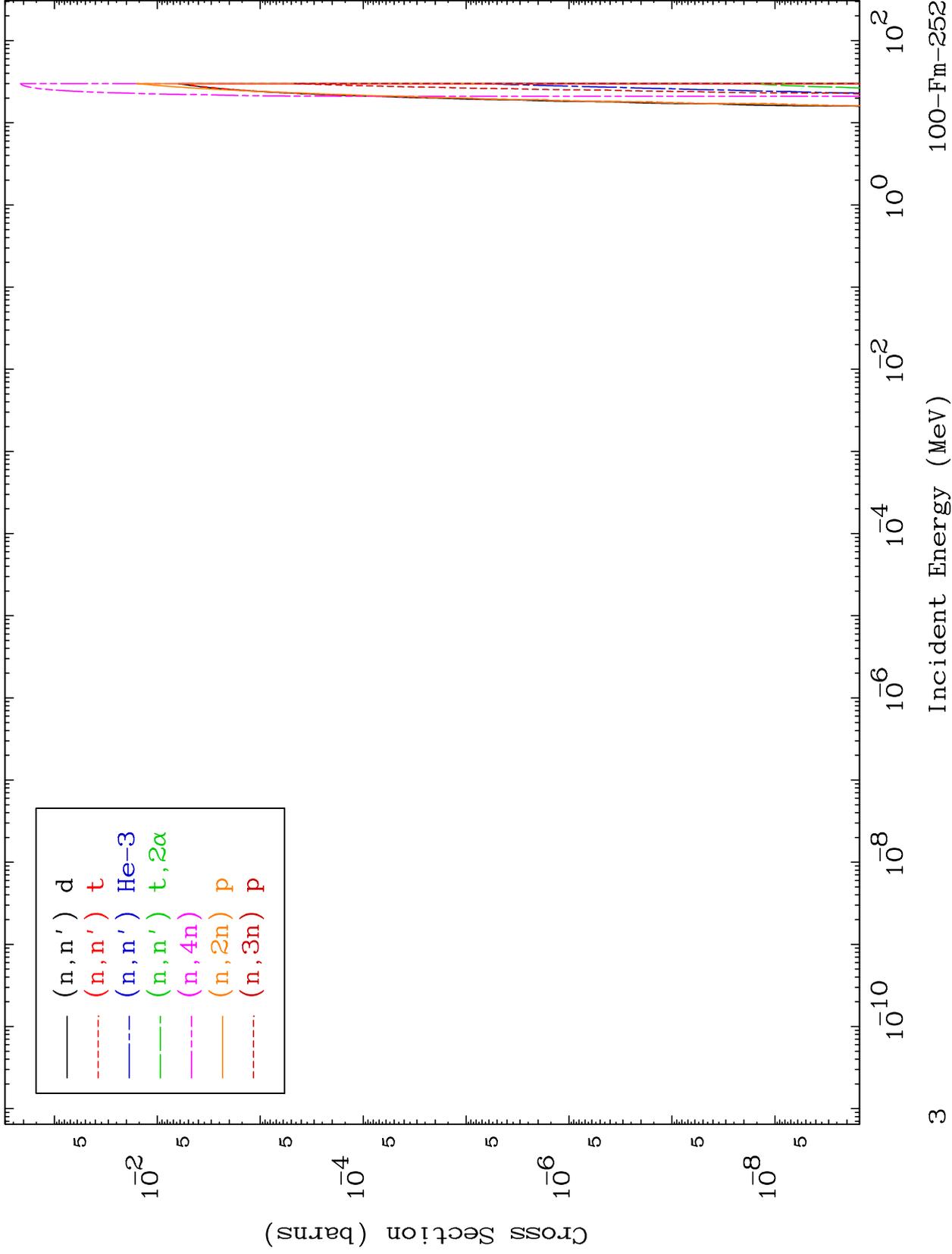
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MAT 9933

Neutron Production
293 Kelvin Cross Sections

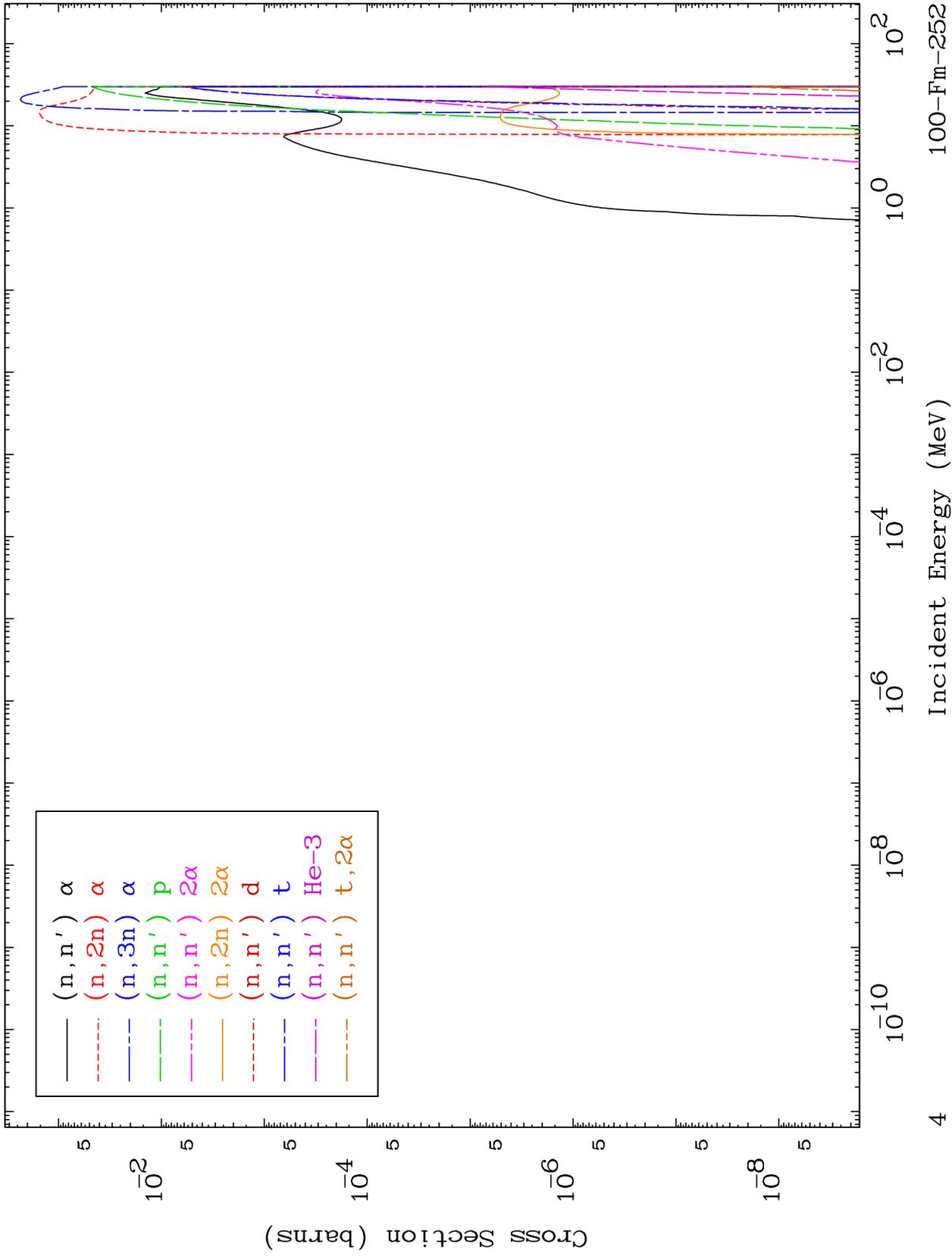
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

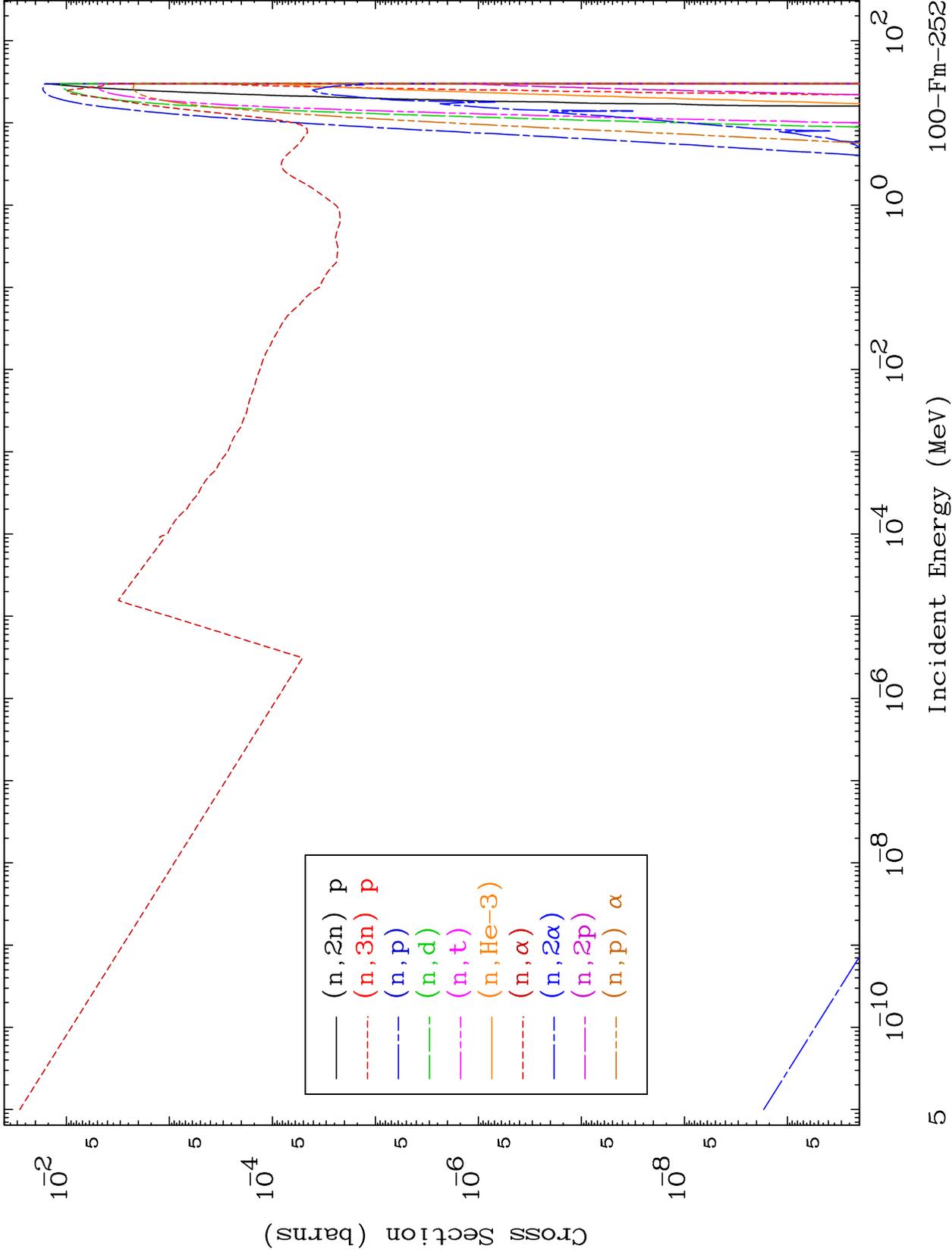
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

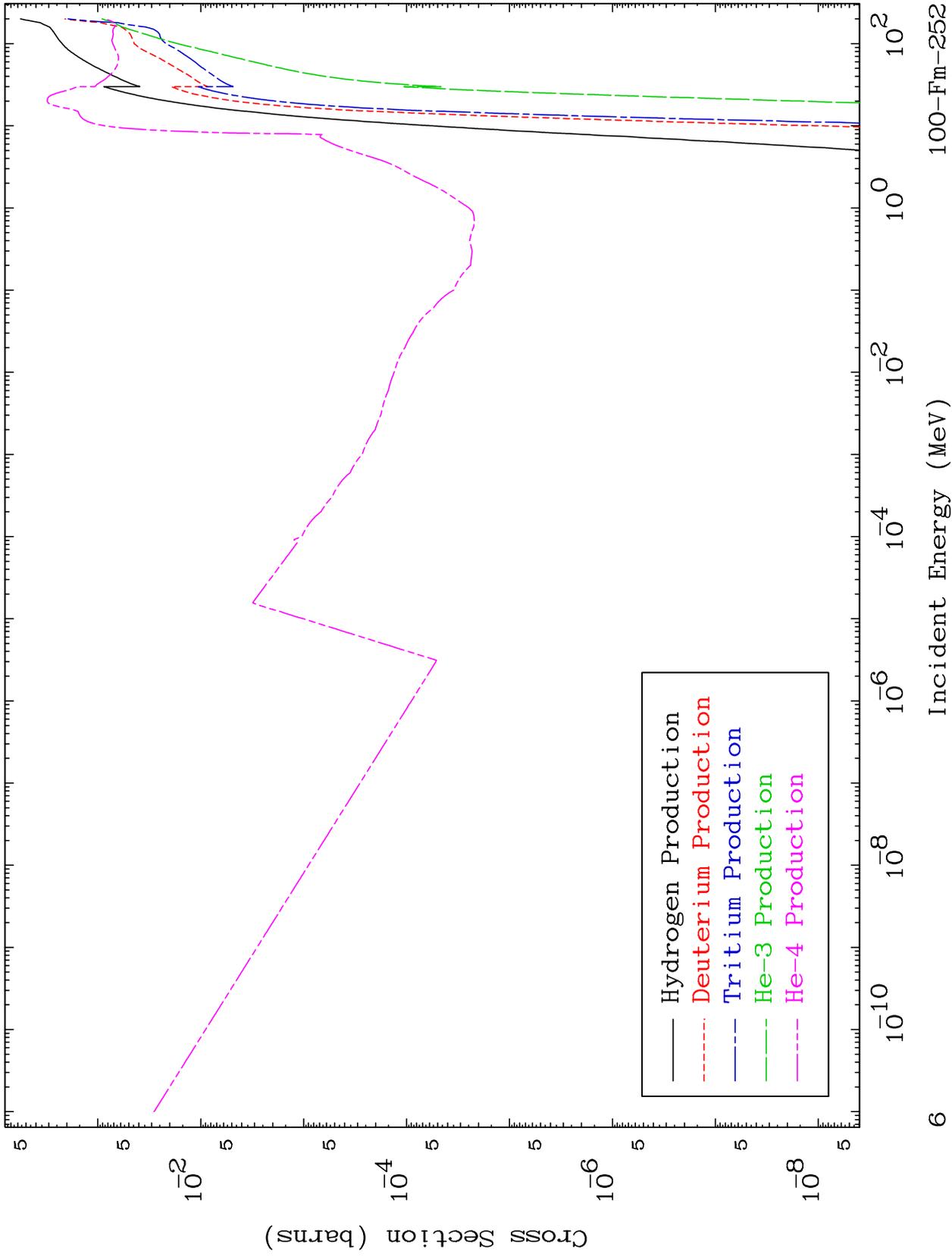
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Particle Production
293 Kelvin Cross Sections

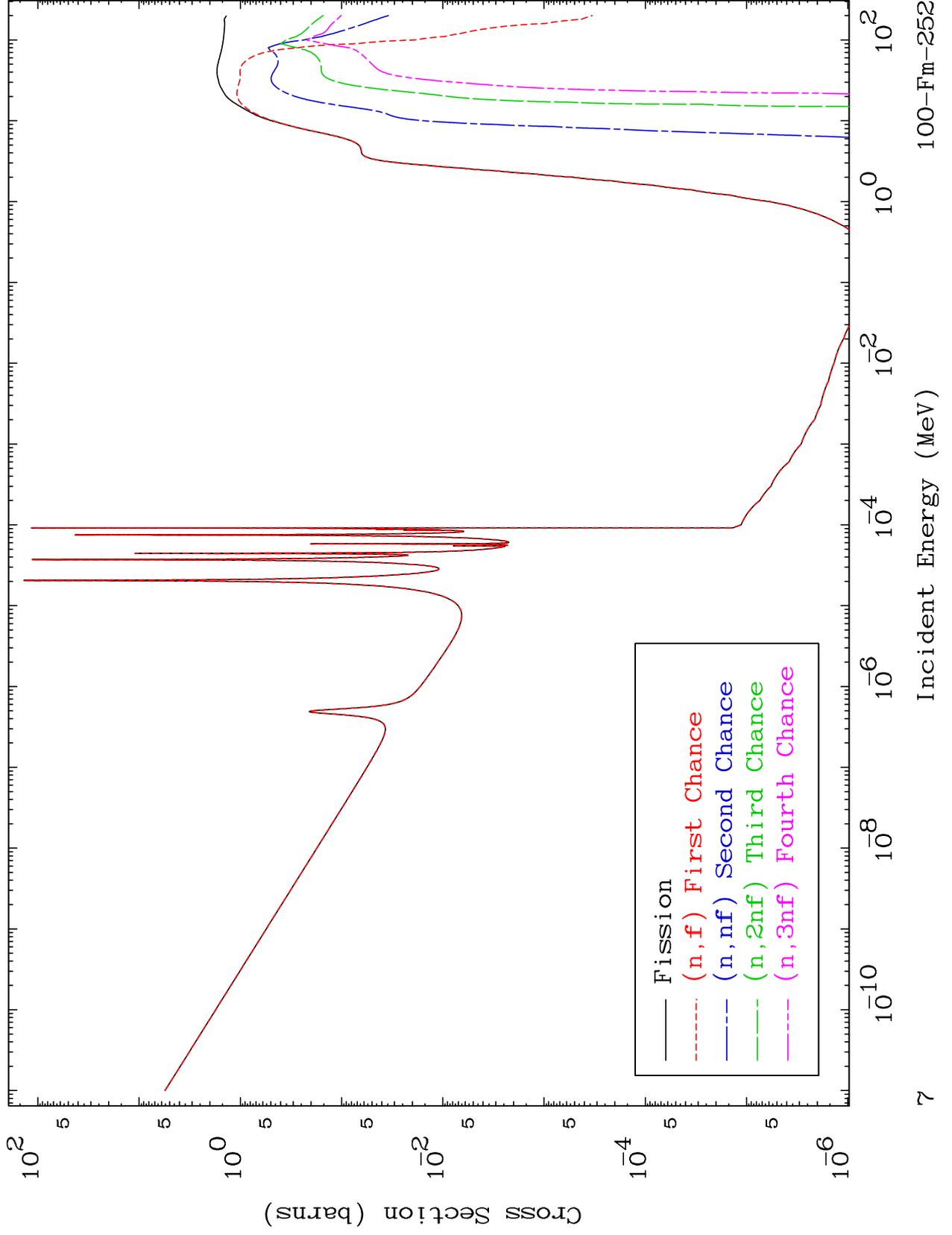
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Fission
293 Kelvin Cross Sections

100-Fm-252

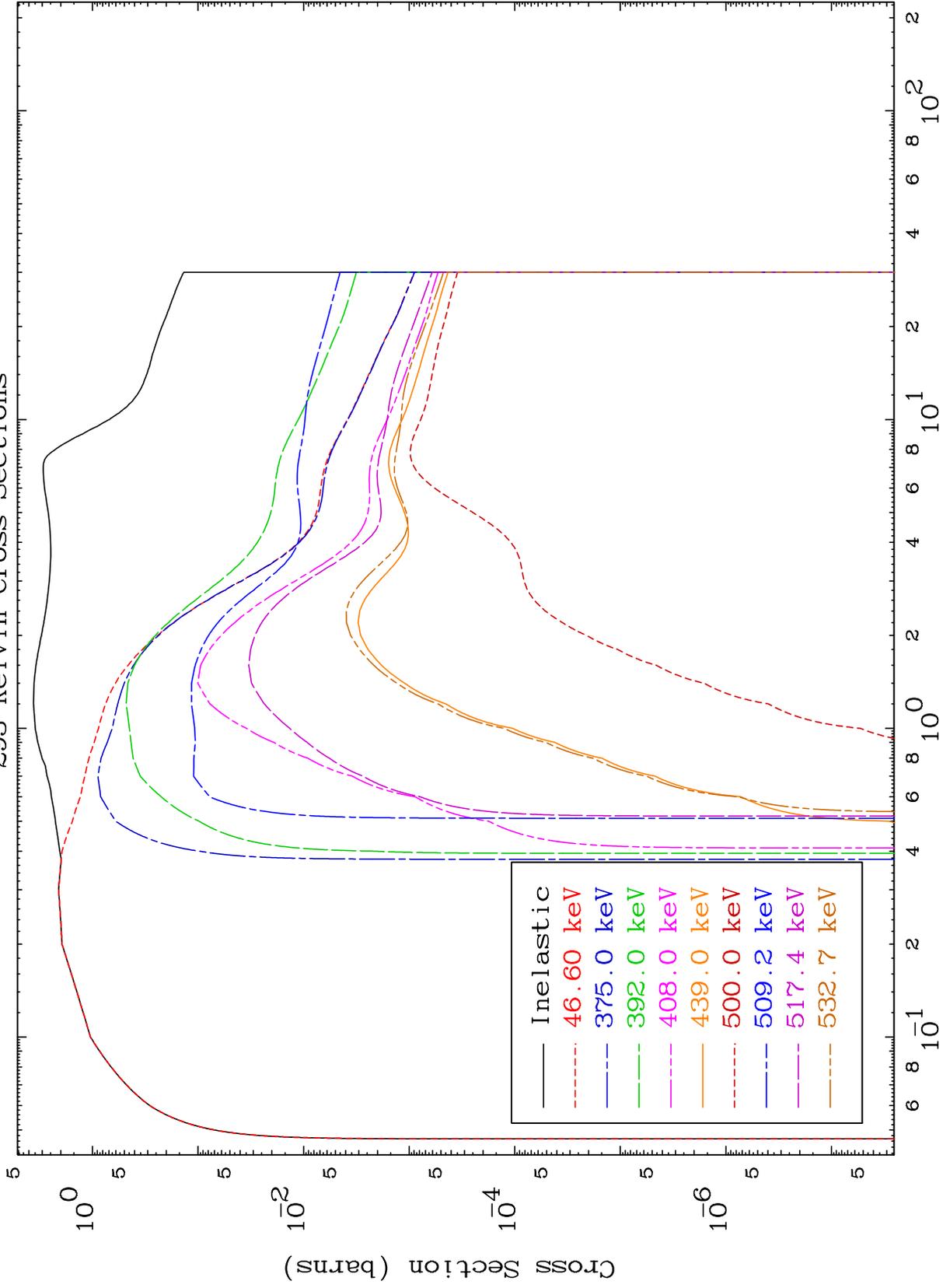


MAT 9933

(n,n') Level

100-Fm-252

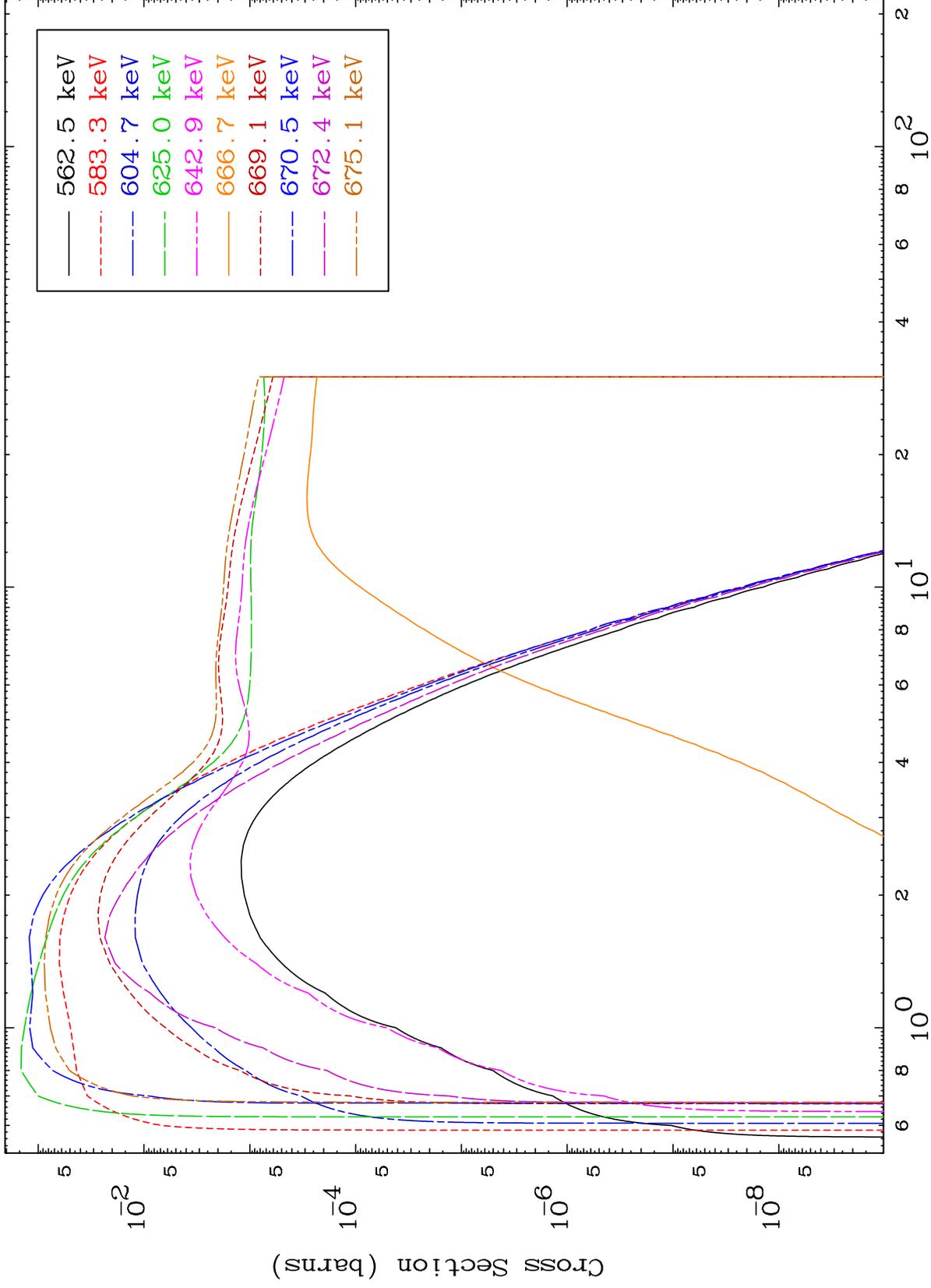
293 Kelvin Cross Sections



MAT 9933

(n,n') Level
293 Kelvin Cross Sections

100-Fm-252



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Incident Energy (MeV)

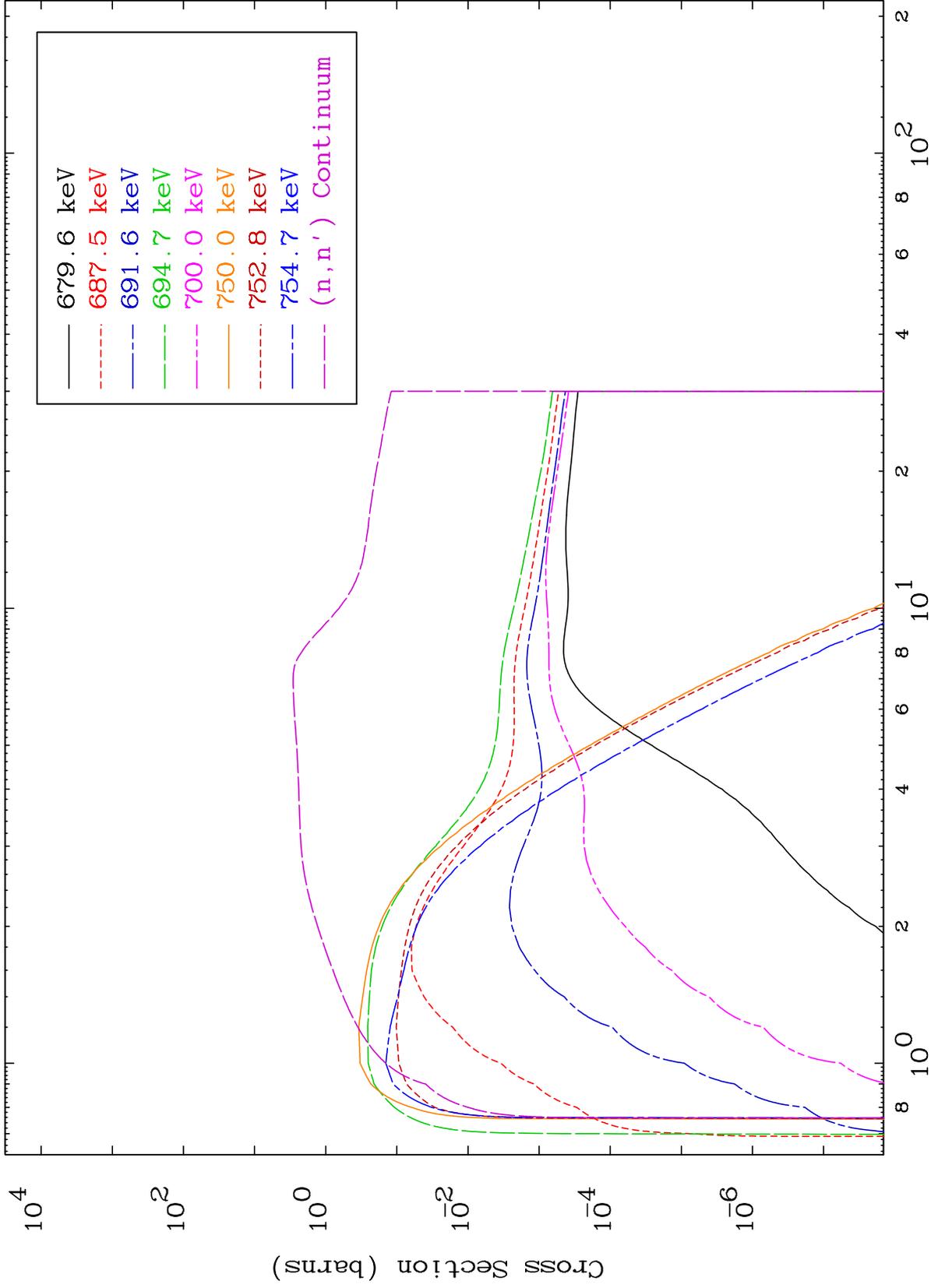
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MAT 9933

(n,n') Level

100-Fm-252

293 Kelvin Cross Sections



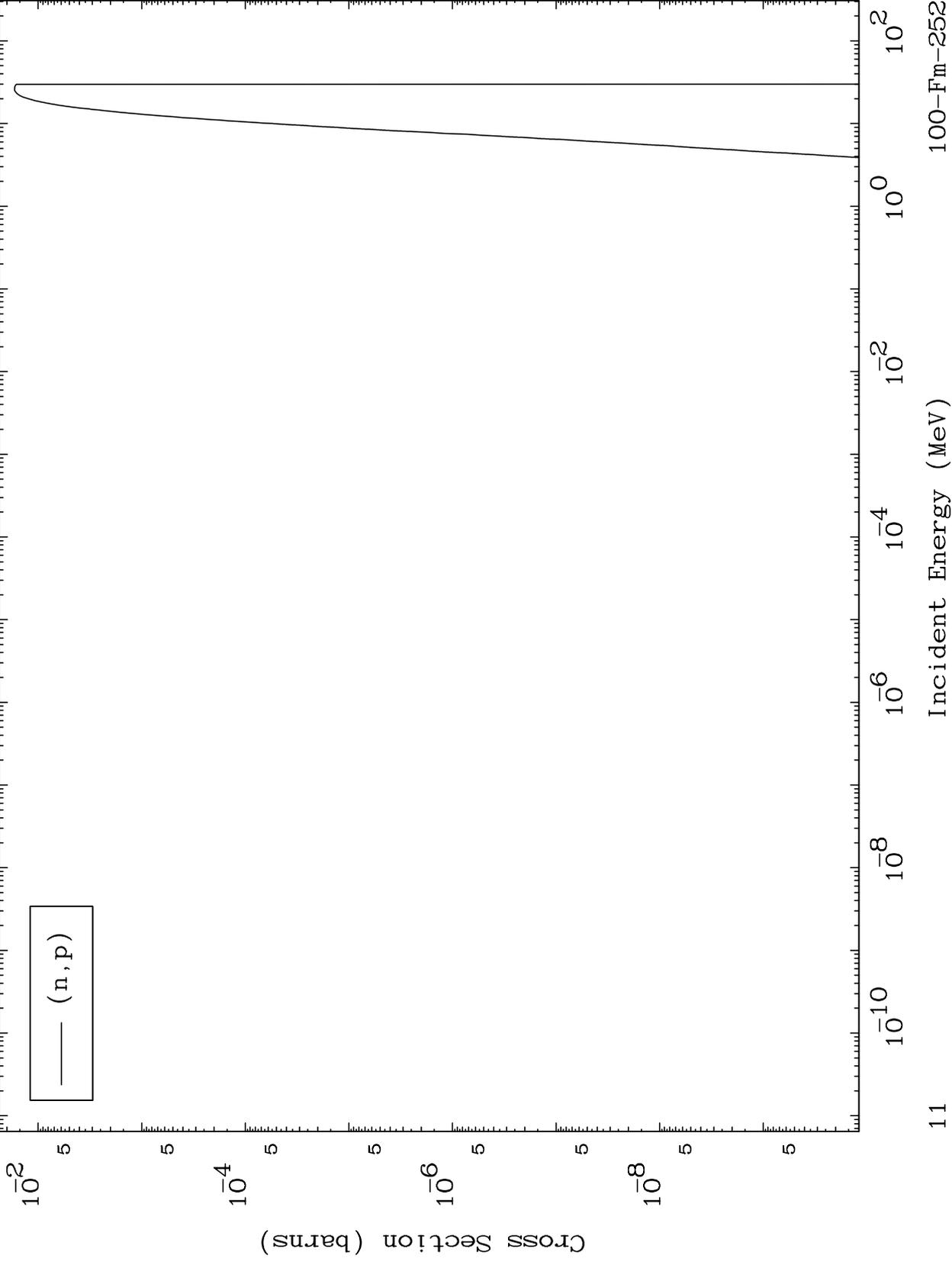
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,p) Levels
293 Kelvin Cross Sections

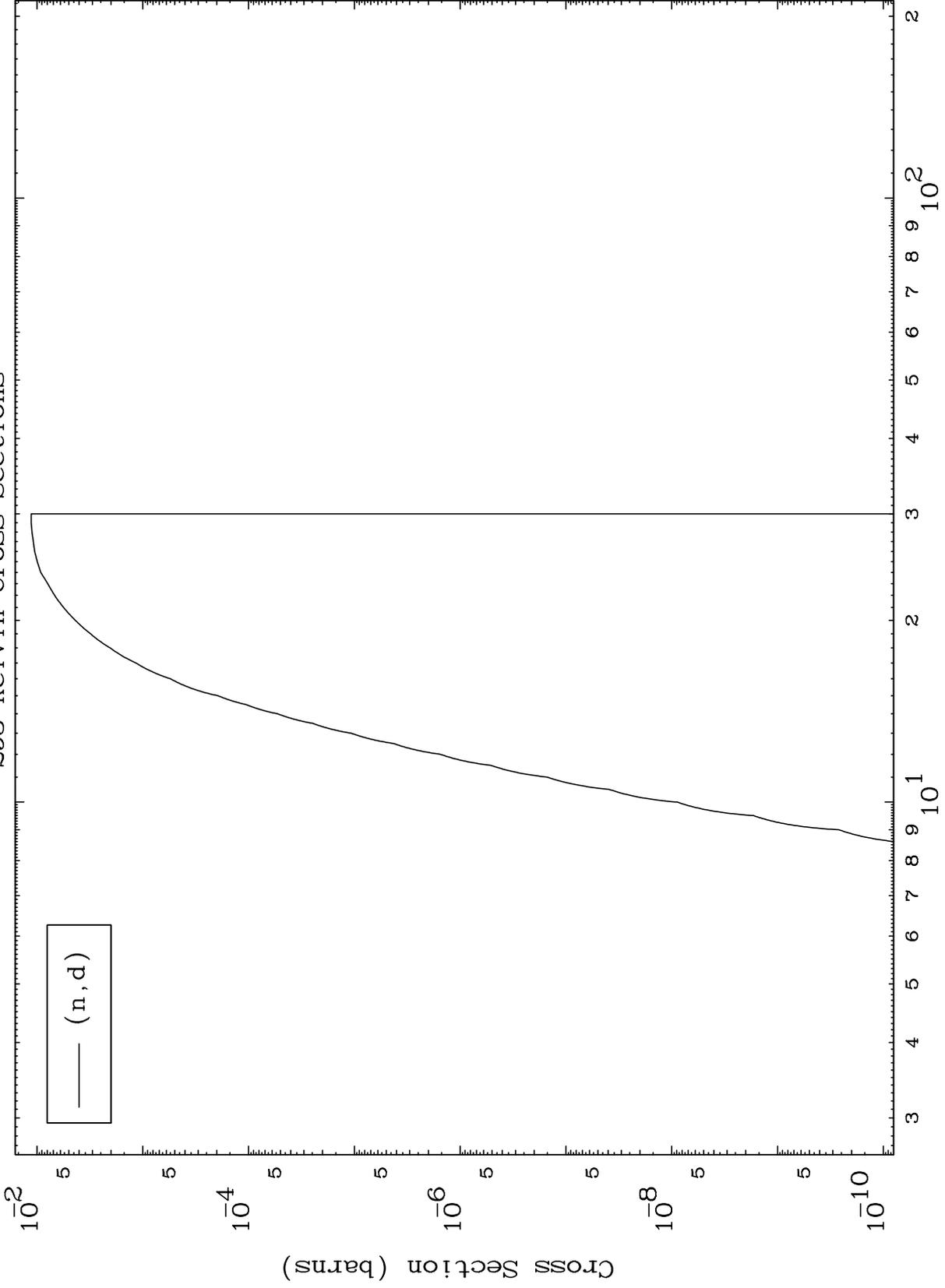
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MAT 9933

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-252



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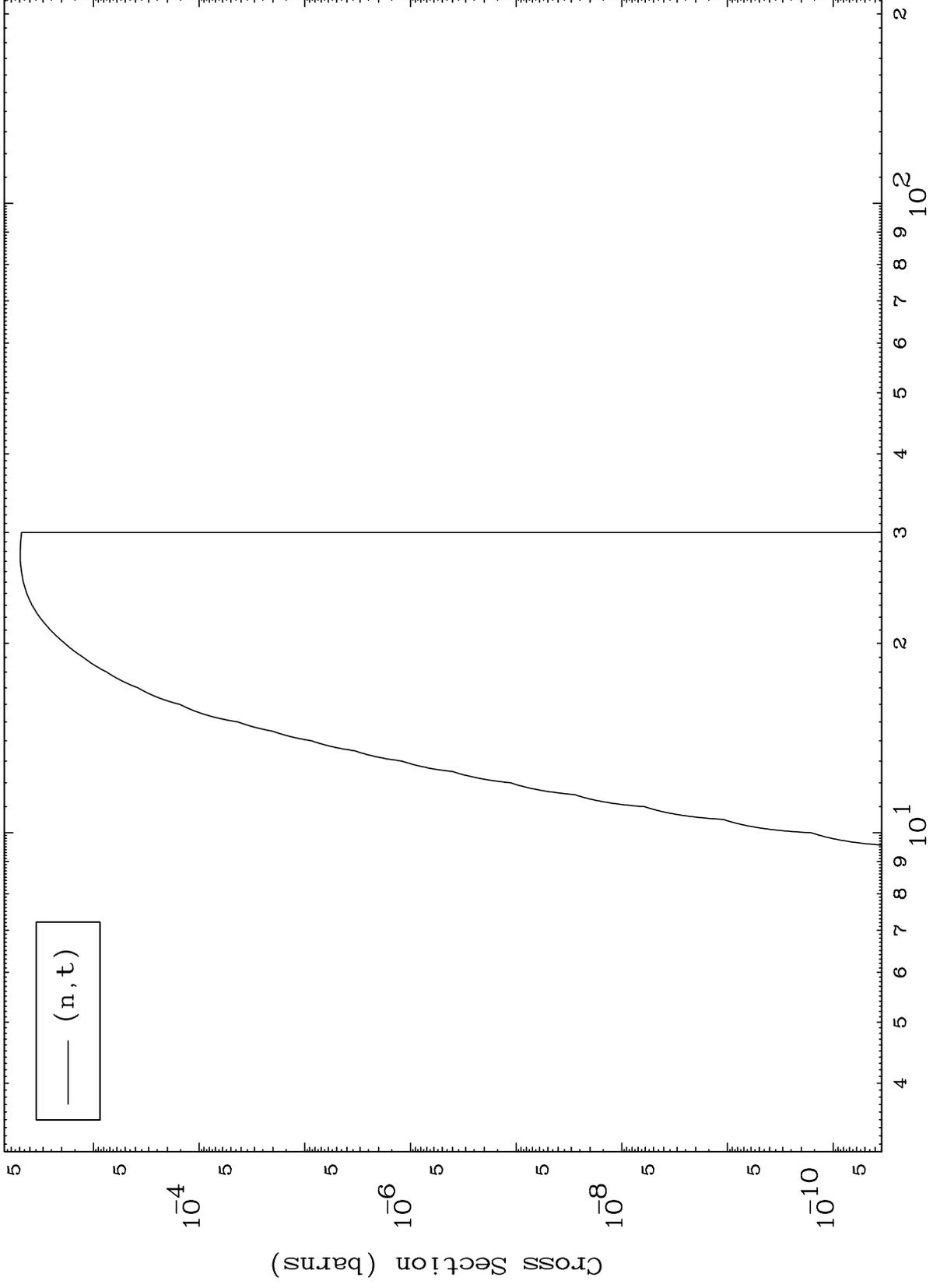
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100-Fm-252

MAT 9933

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-252



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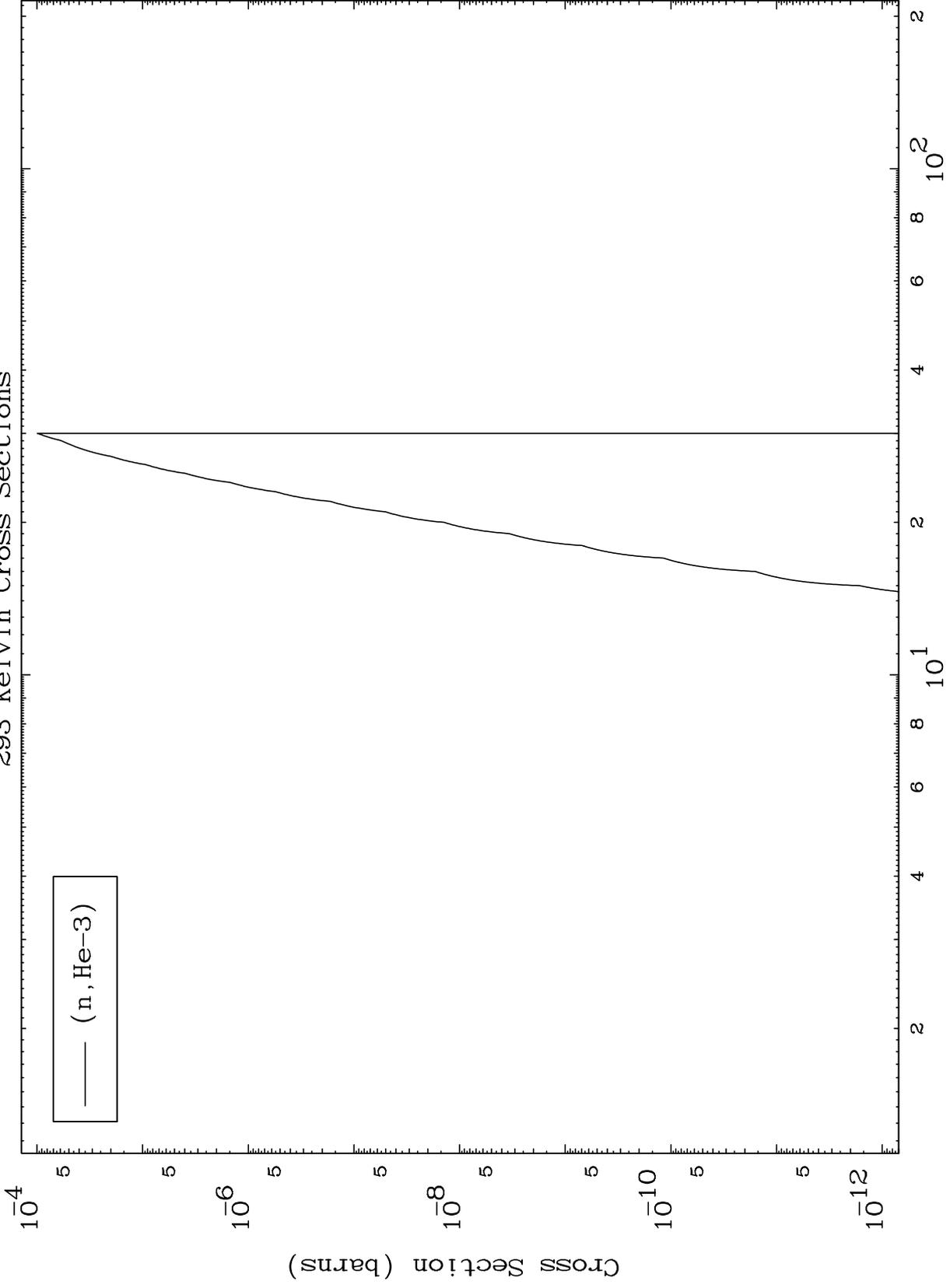
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,He3) Levels
293 Kelvin Cross Sections

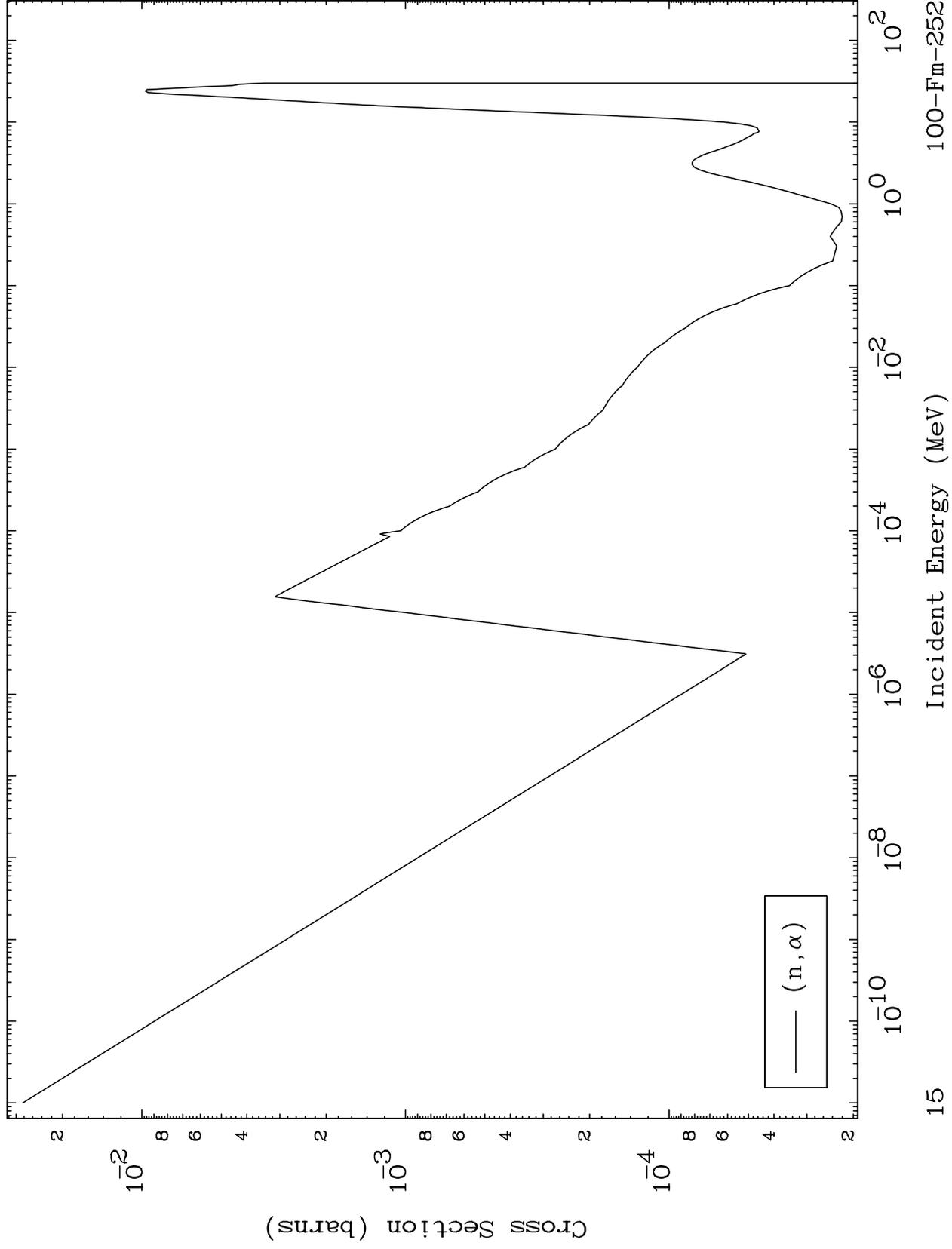
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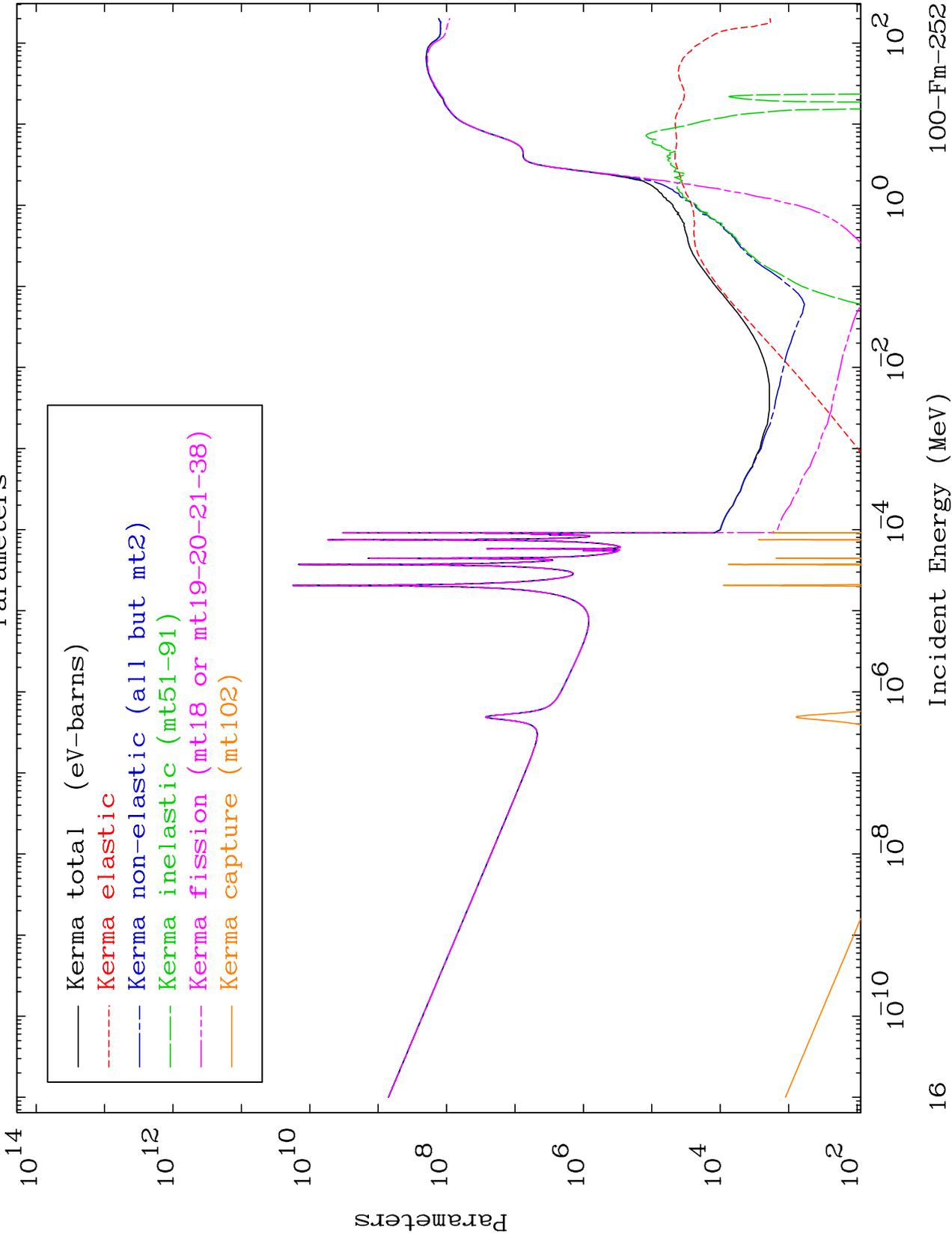
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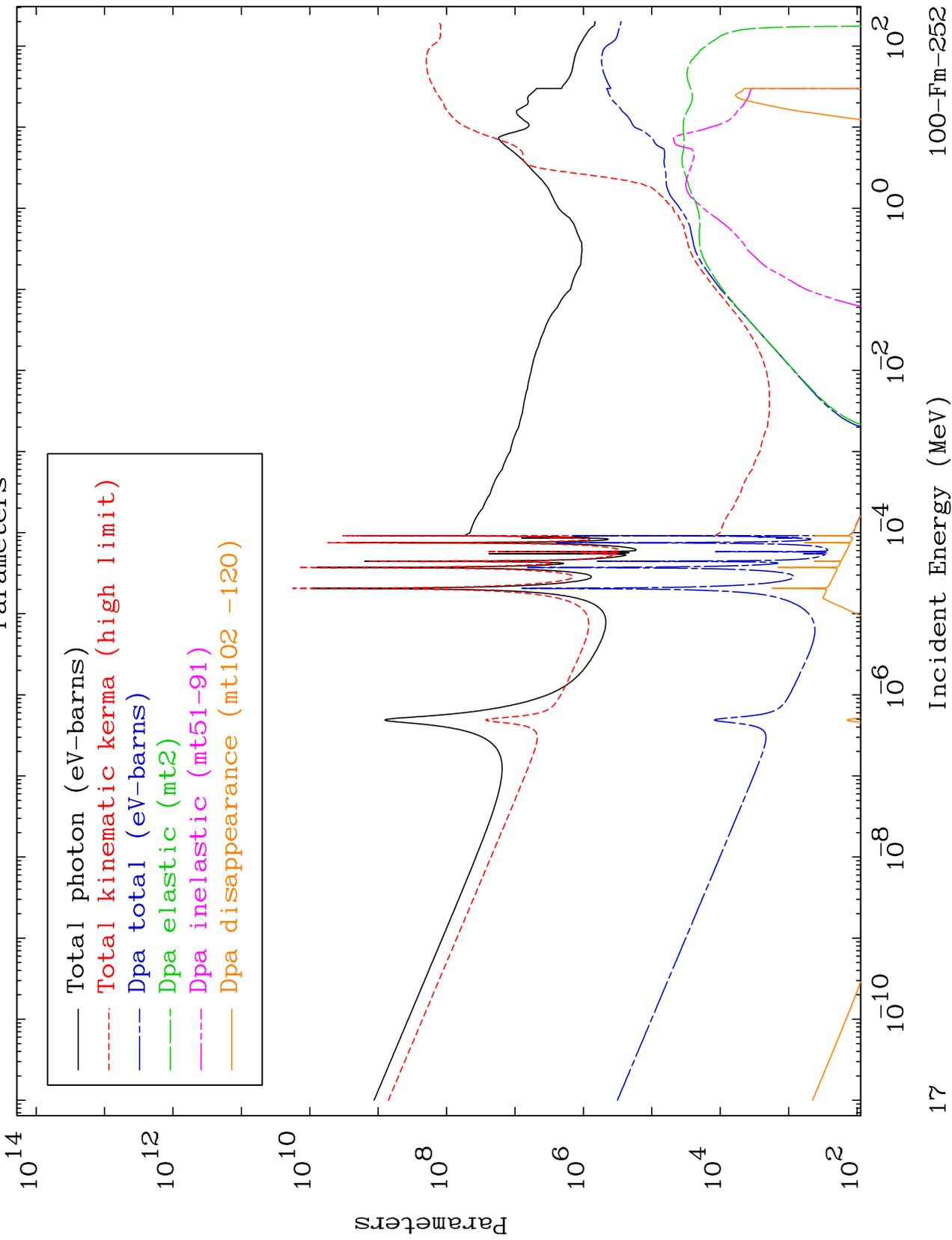
(n,α) Levels
293 Kelvin Cross Sections

100-Fm-252



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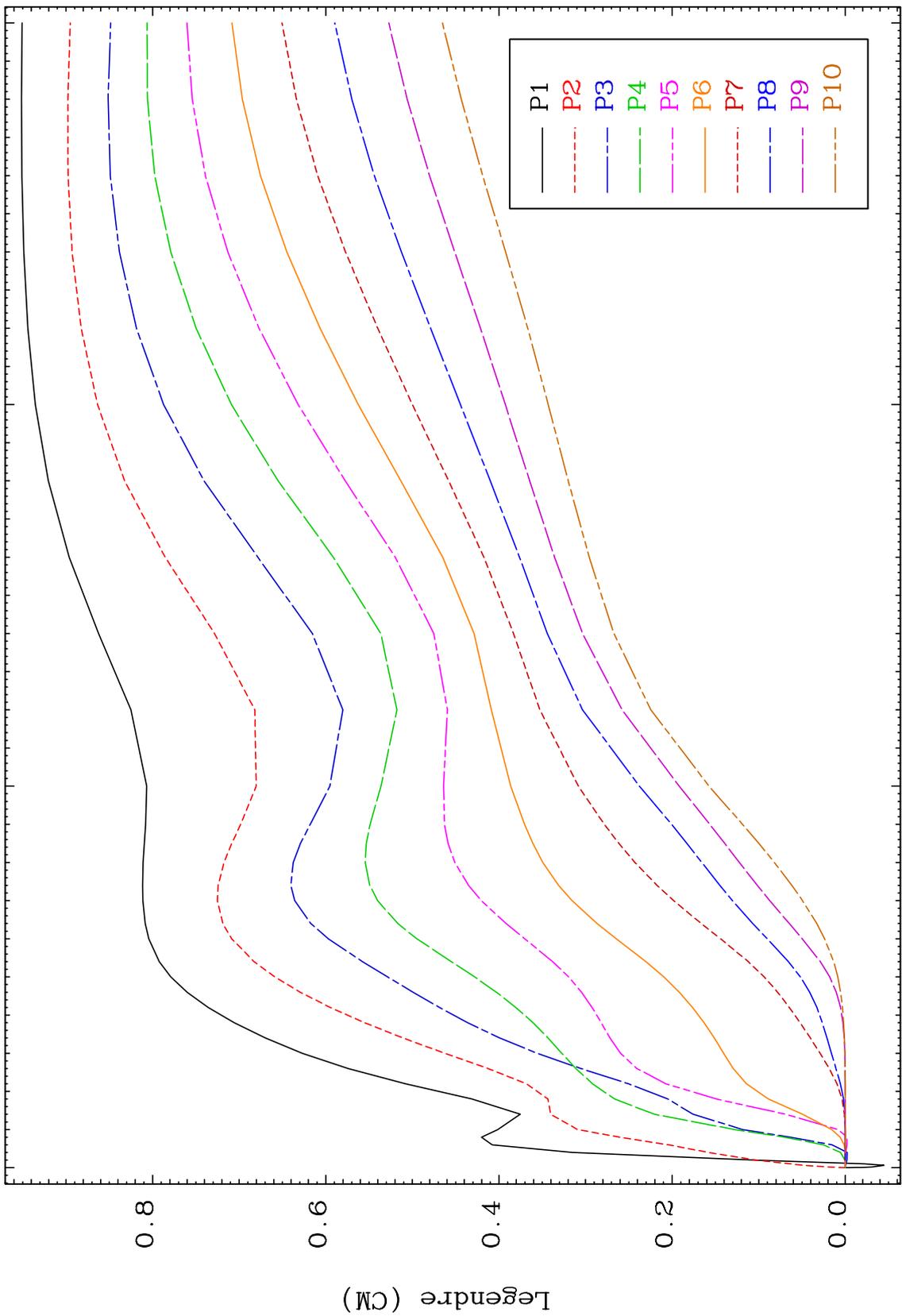




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Elastic Legendre Coefficients

100-Fm-252



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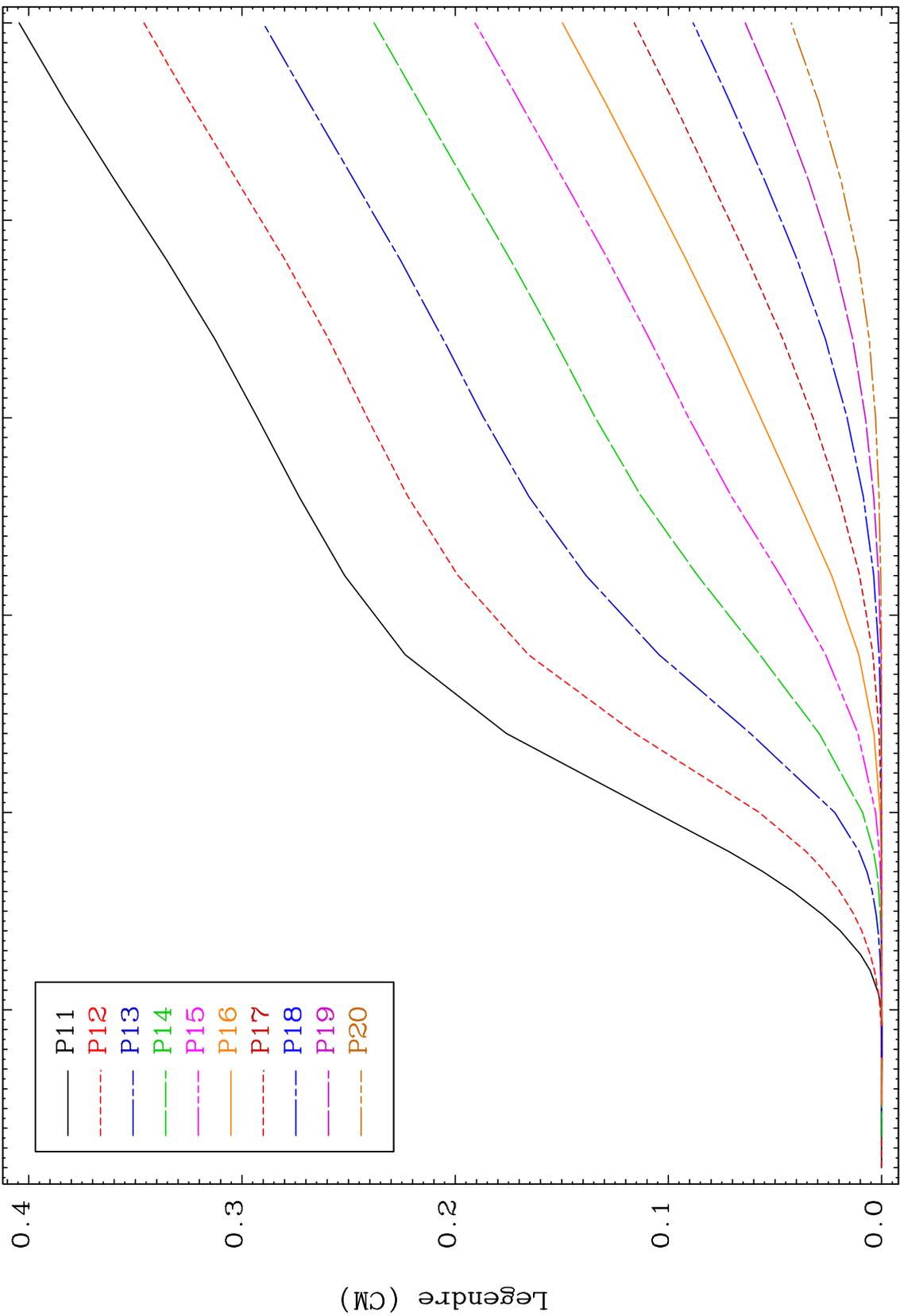
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic Legendre Coefficients

100-Fm-252



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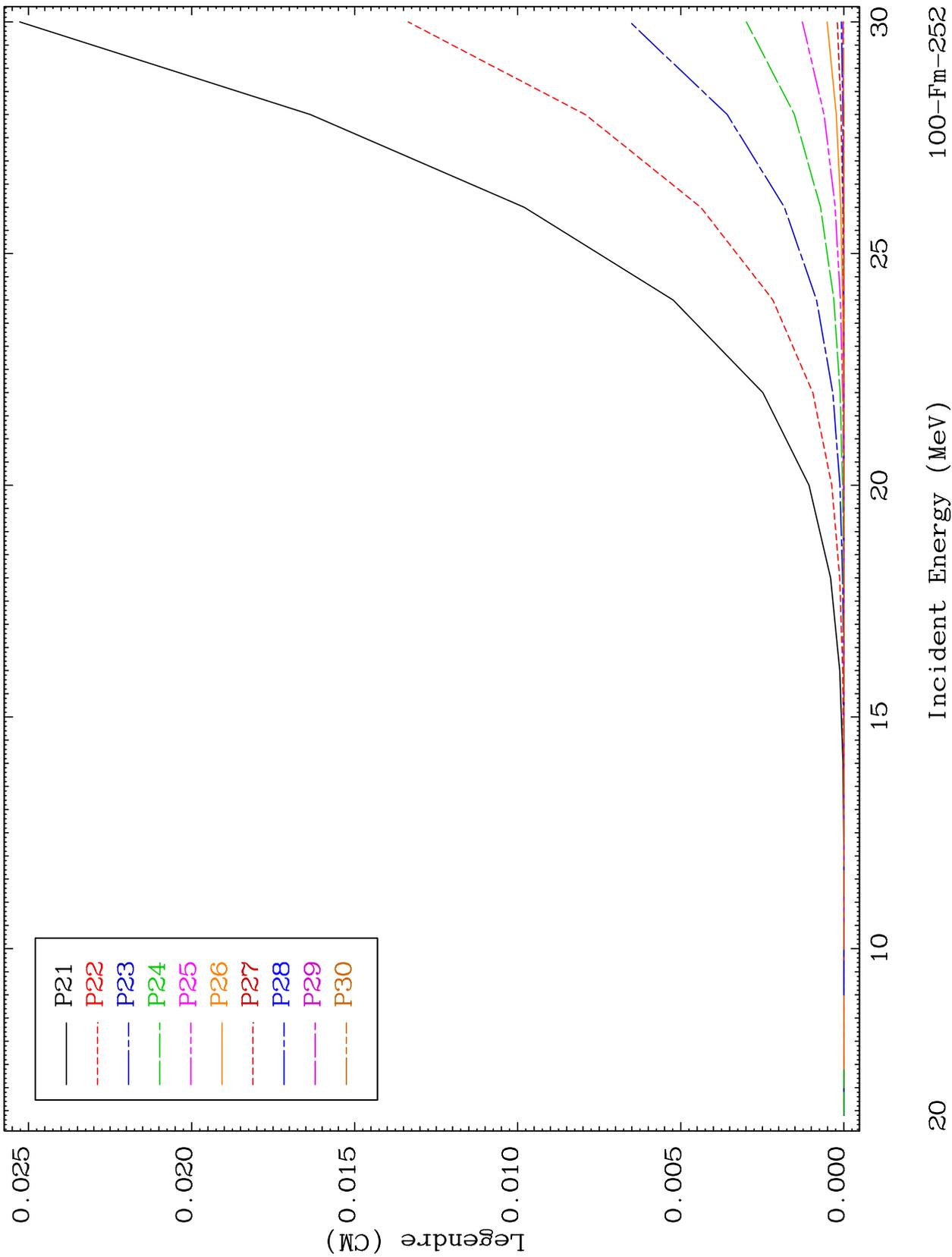
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic
Legendre Coefficients

100-Fm-252



20

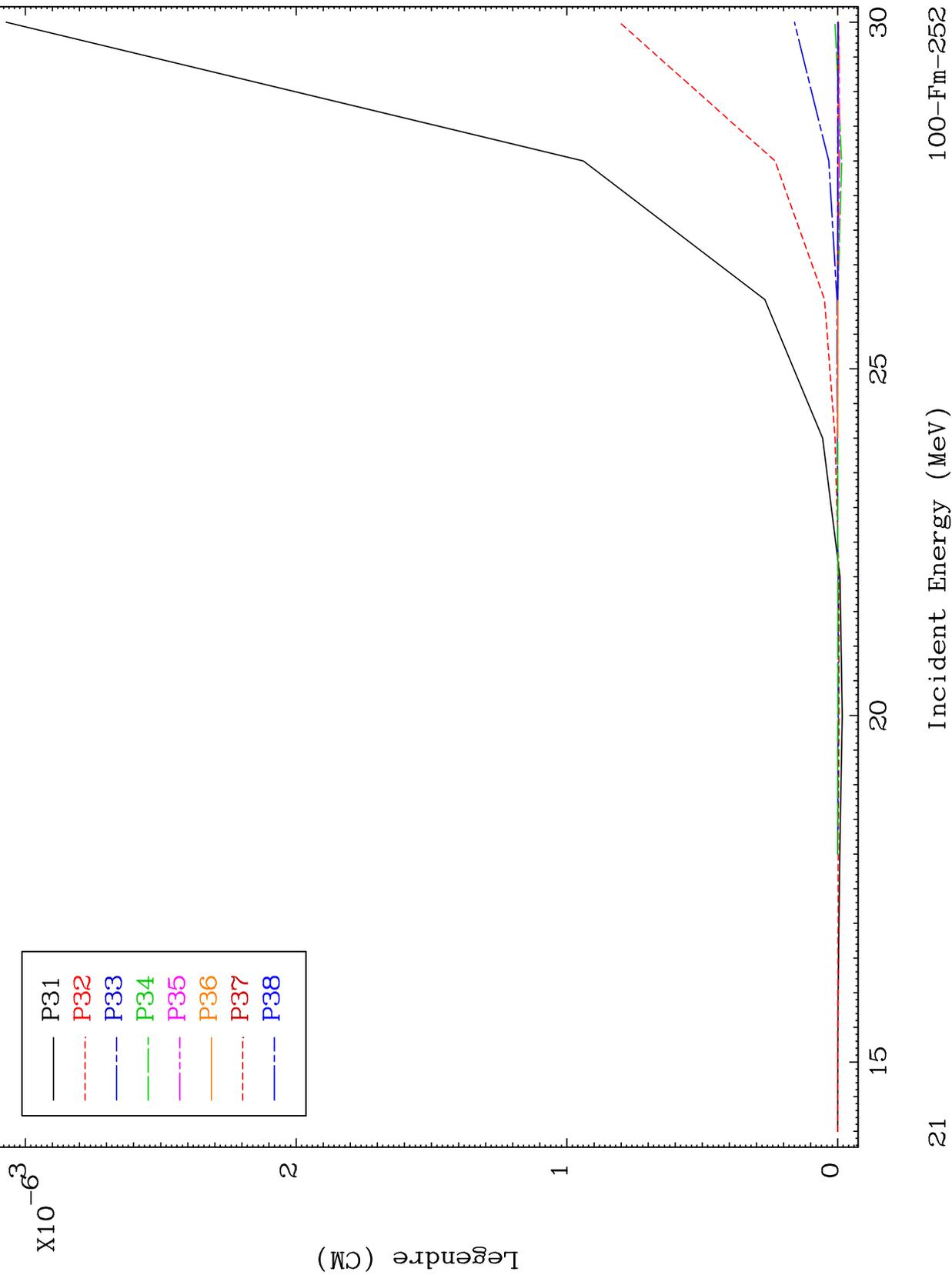
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic
Legendre Coefficients

100-Fm-252

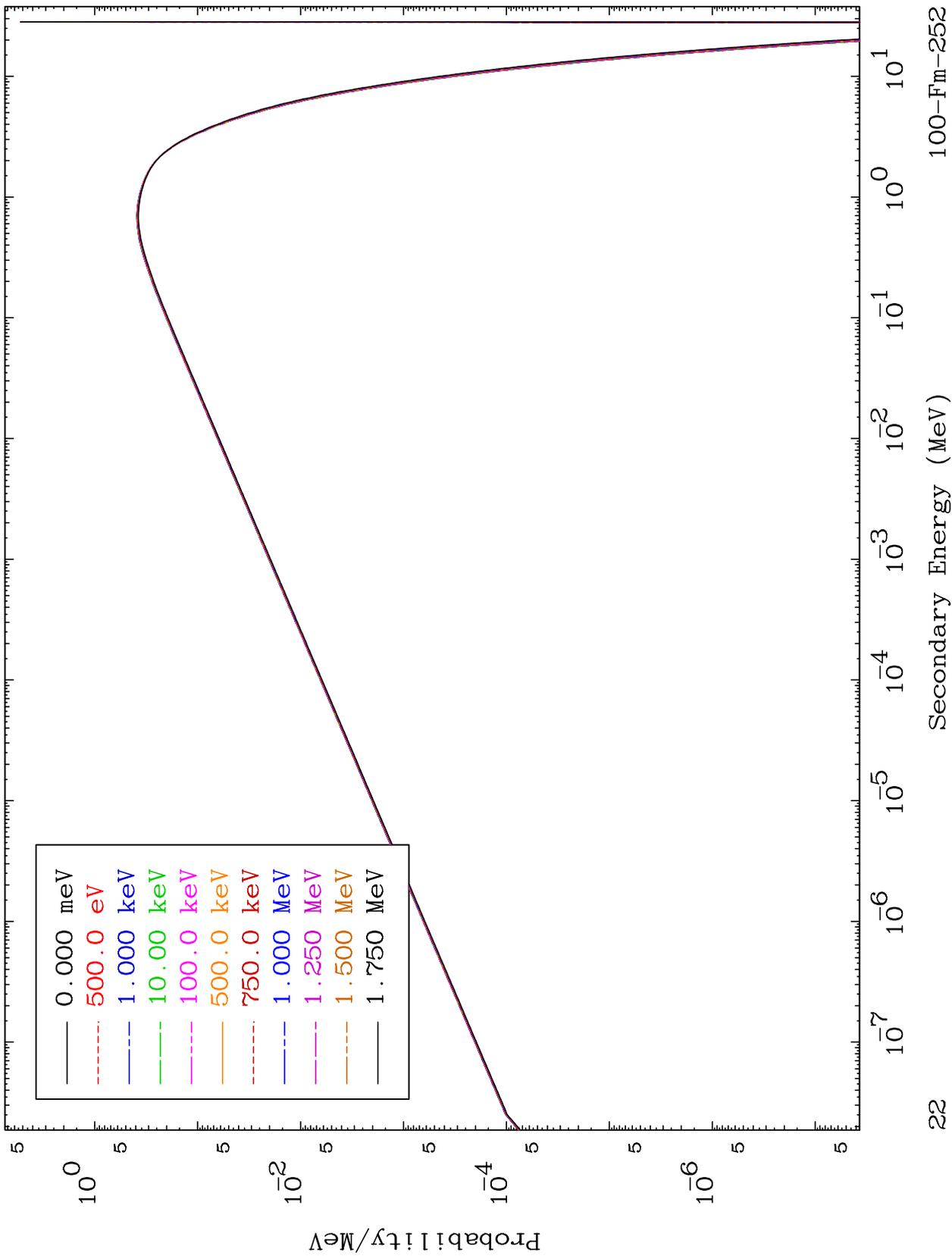


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Incident Energy (MeV)

100-Fm-252

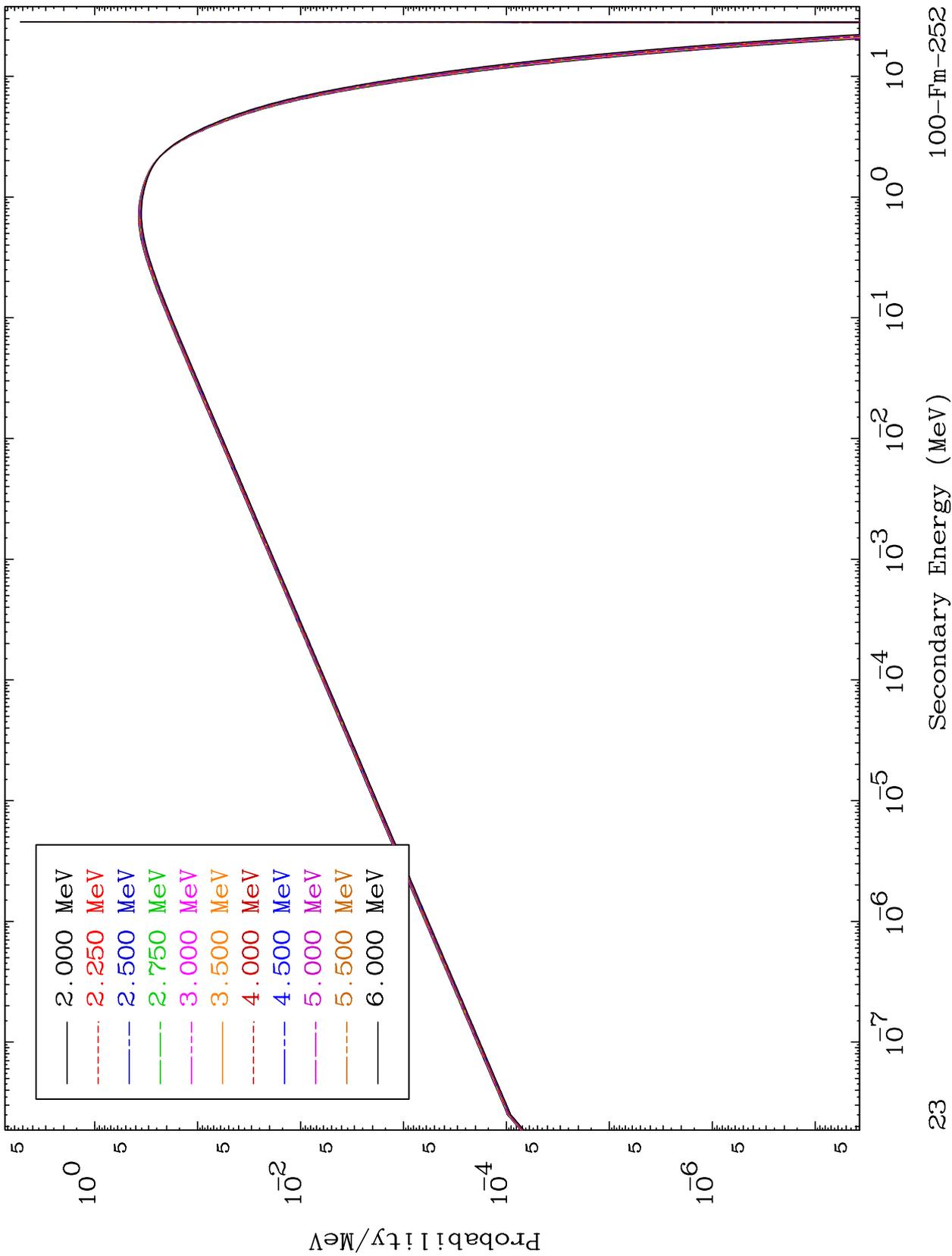
Fission Energy Distribution



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Fission Energy Distribution

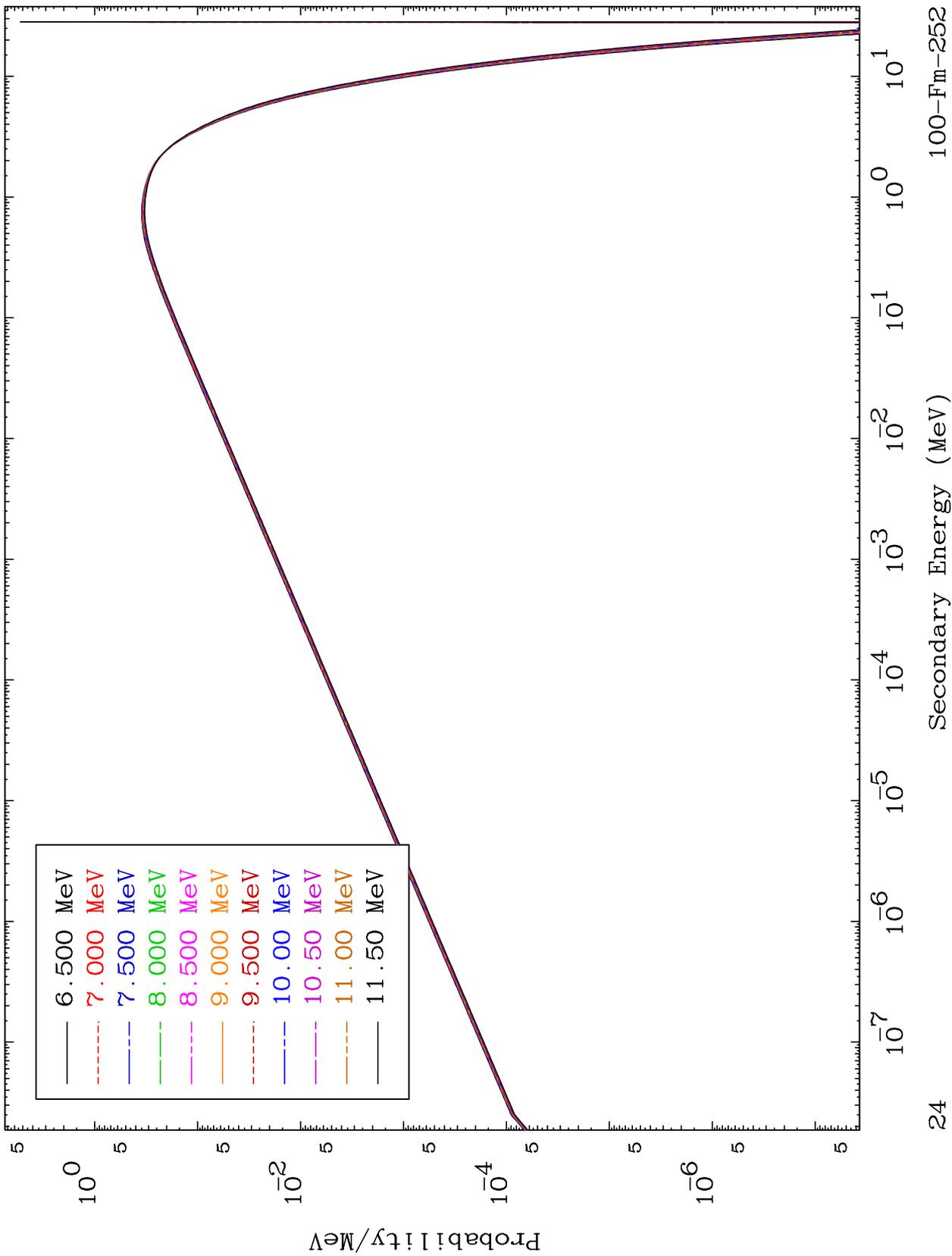
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Fission Energy Distribution

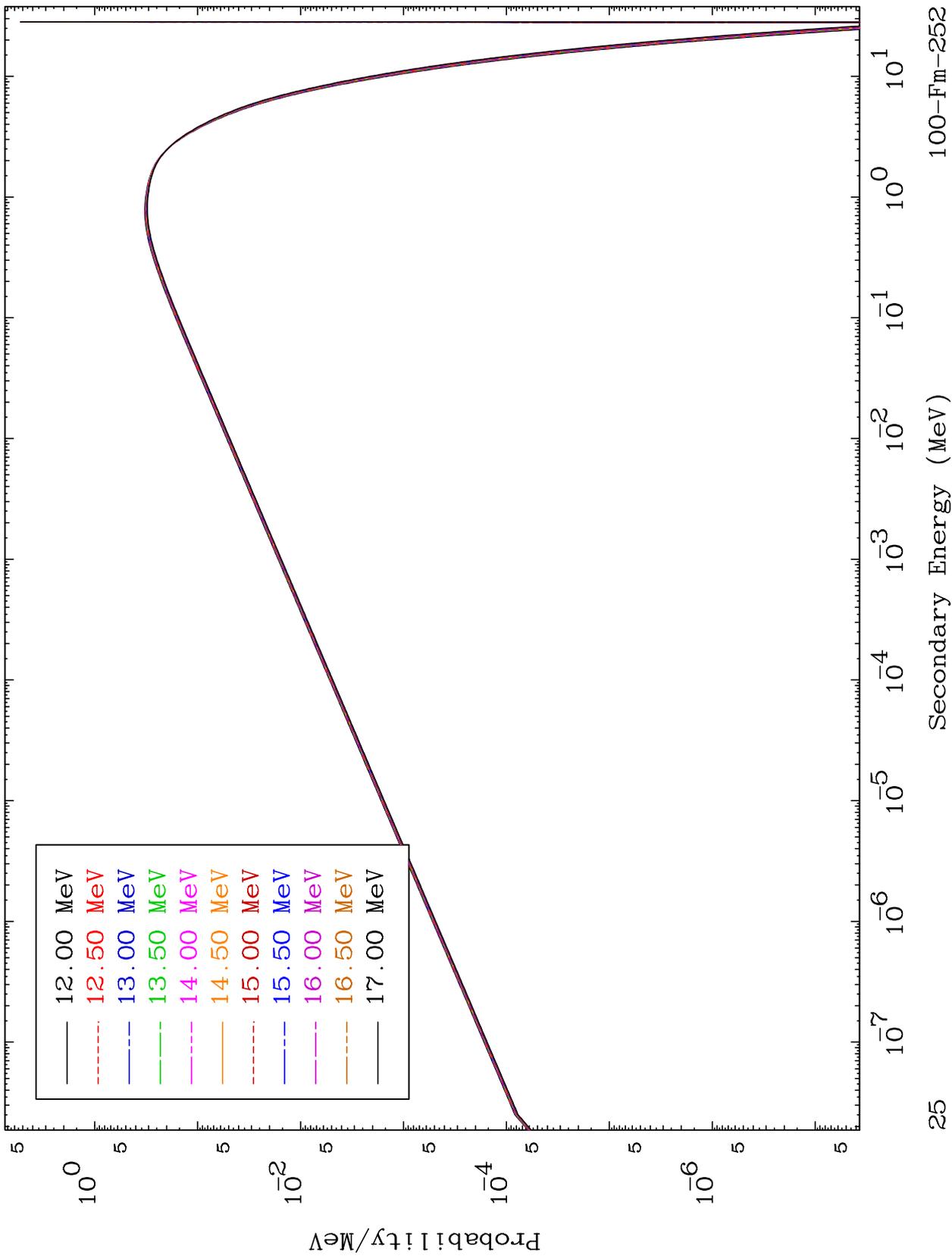
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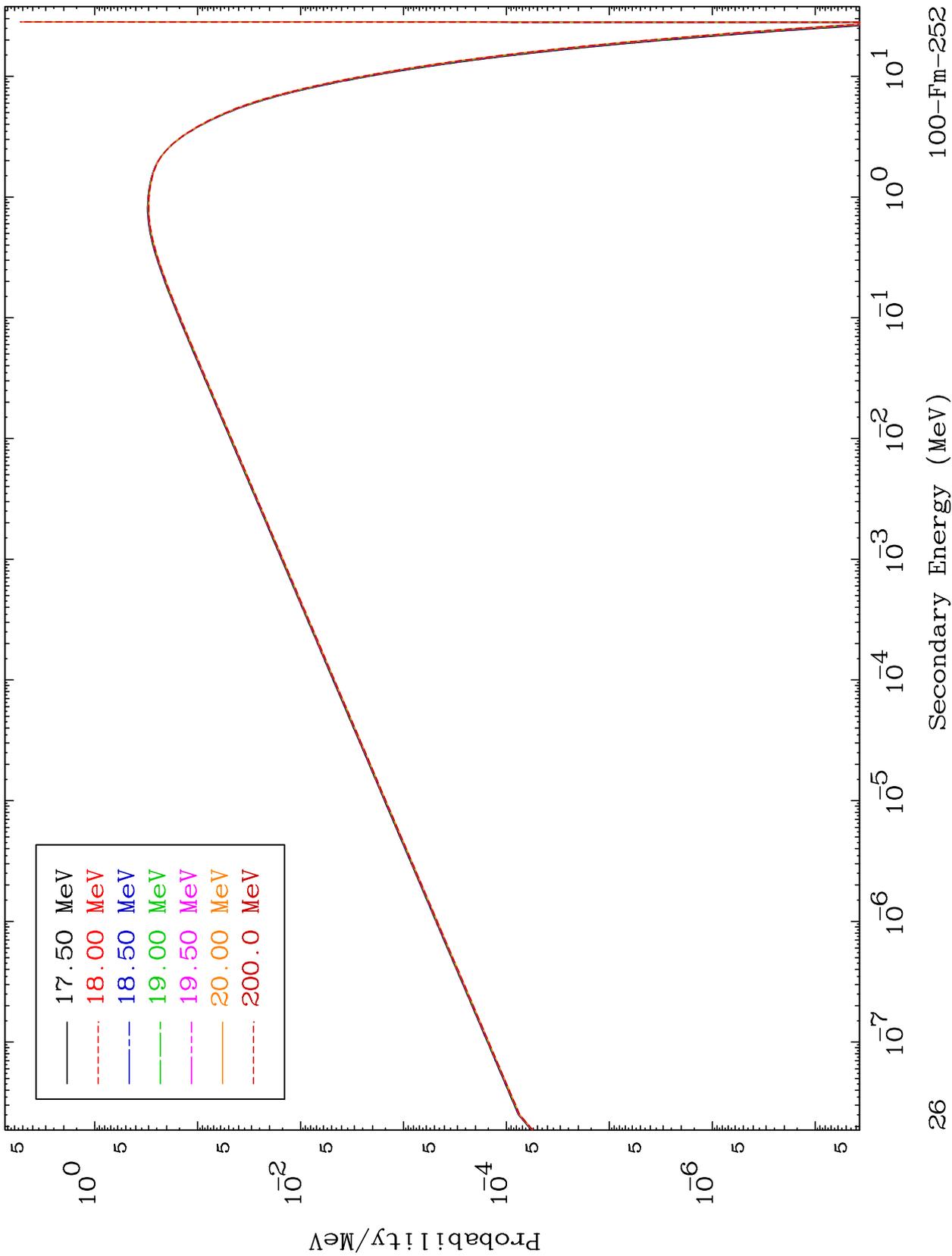
MAT 9933

Fission Energy Distribution

100-Fm-252



Fission
Energy Distribution

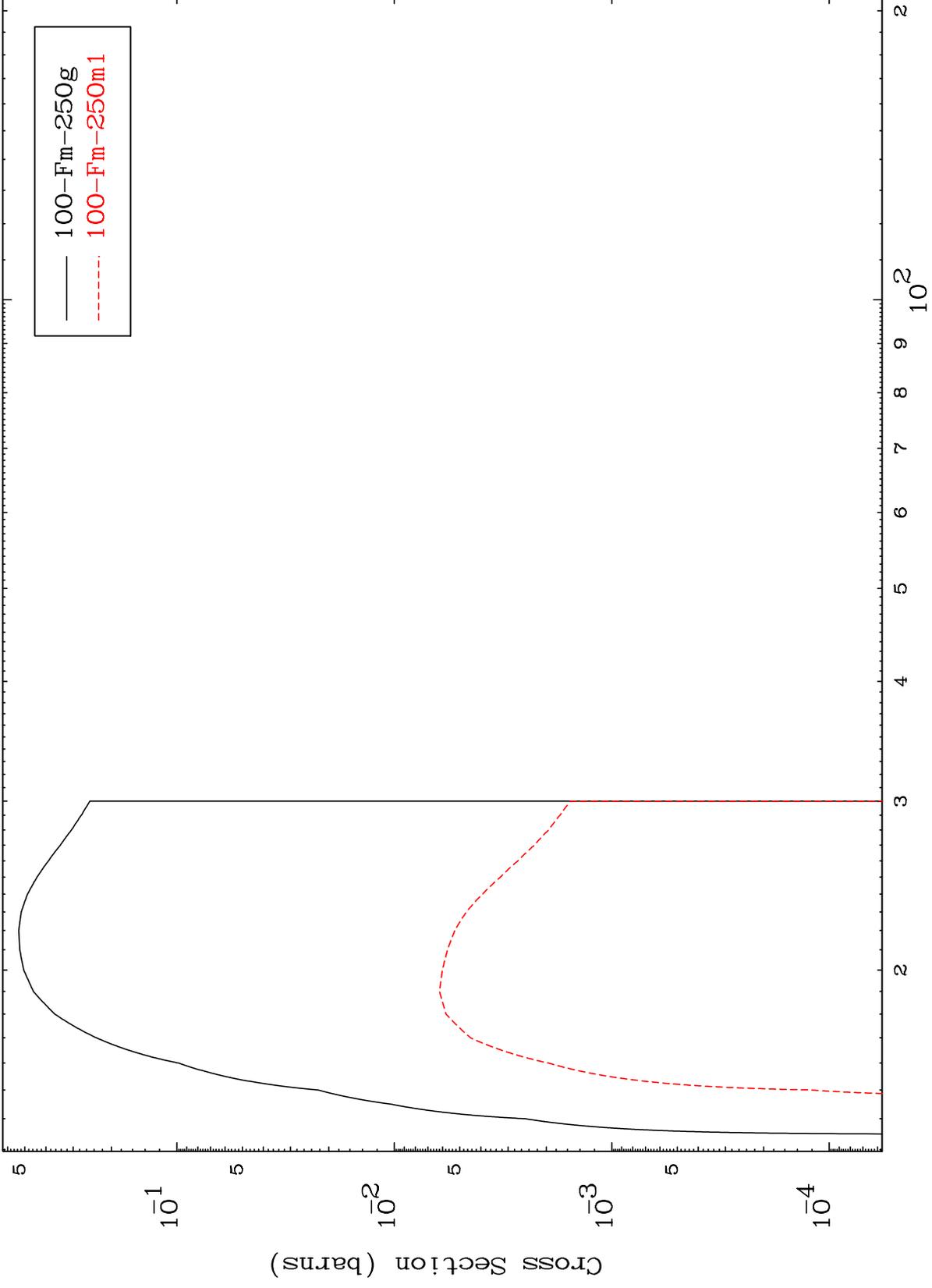


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(n,3n)

100-Fm-252

Radionuclide Production Cross Section



27

Incident Energy (MeV)

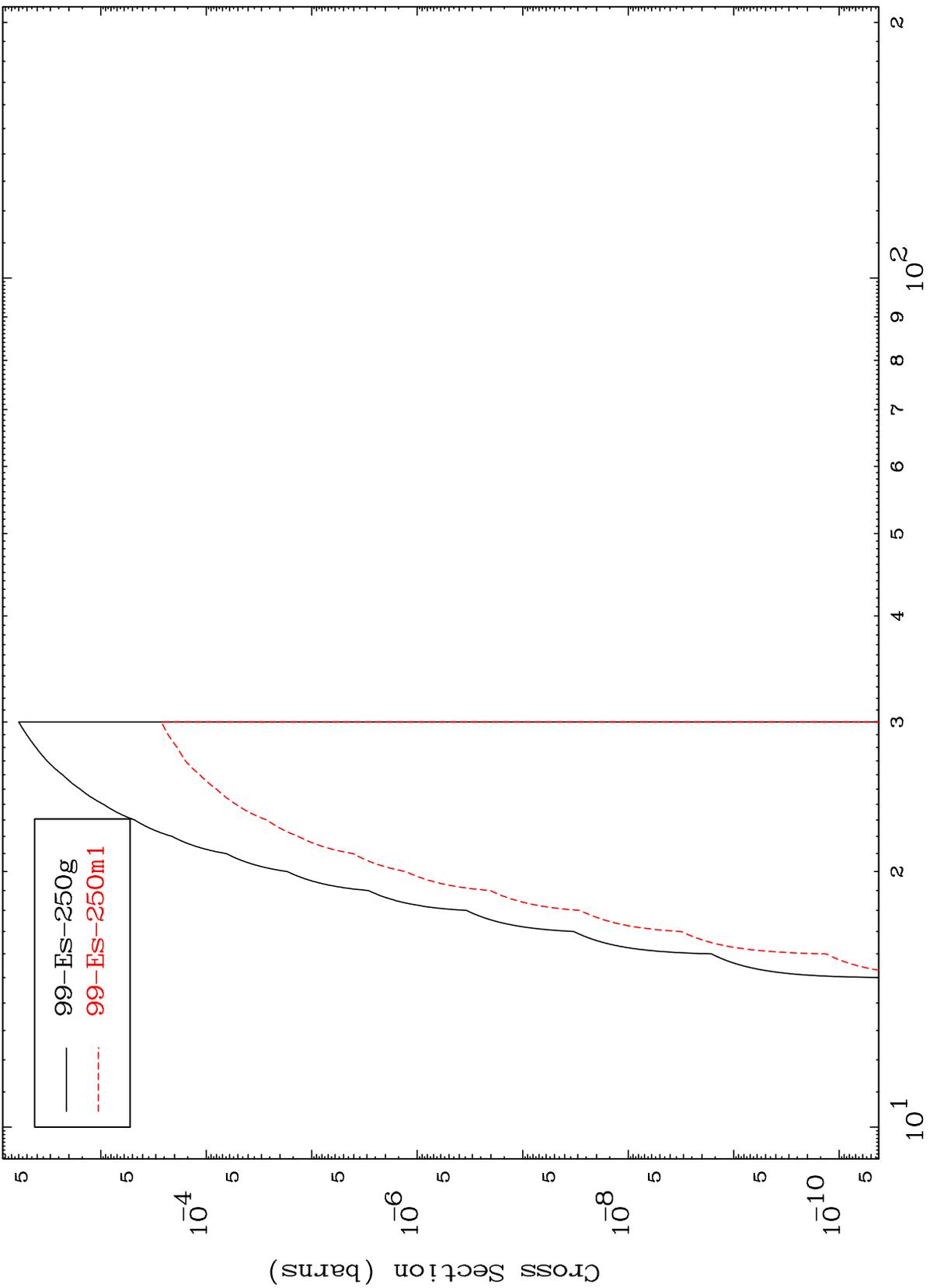
100-Fm-252

MAT 9933

(n,n') d

100-Fm-252

Radionuclide Production Cross Section



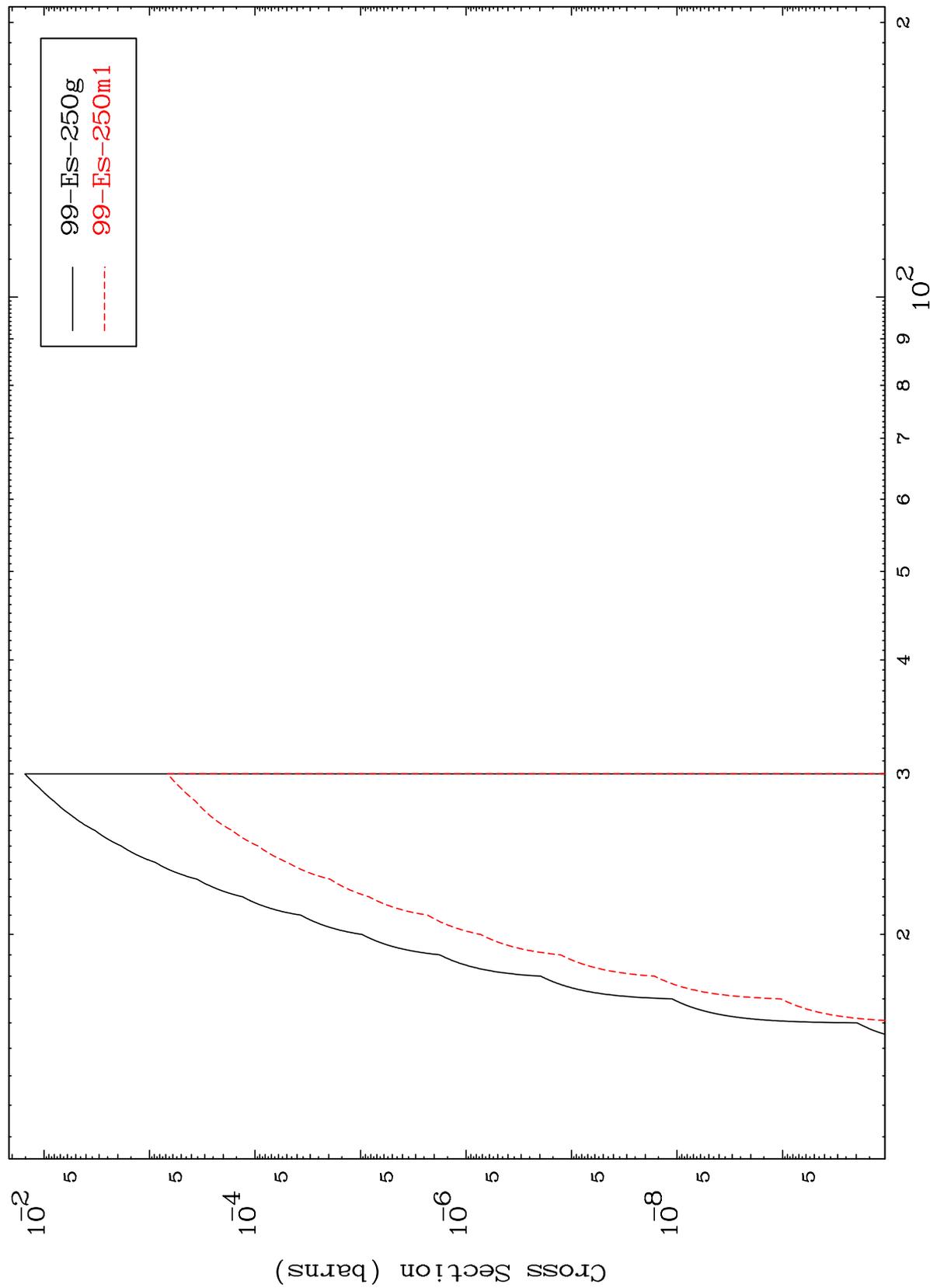
99-Es-250g
99-Es-250m1

Incident Energy (MeV)

100-Fm-252

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Radionuclide Production Cross Section



MAT 9933

(n,p) α

100-Fm-252

Radionuclide Production Cross Section

