

Program EVALPLOT
(Version 2015-2)

by

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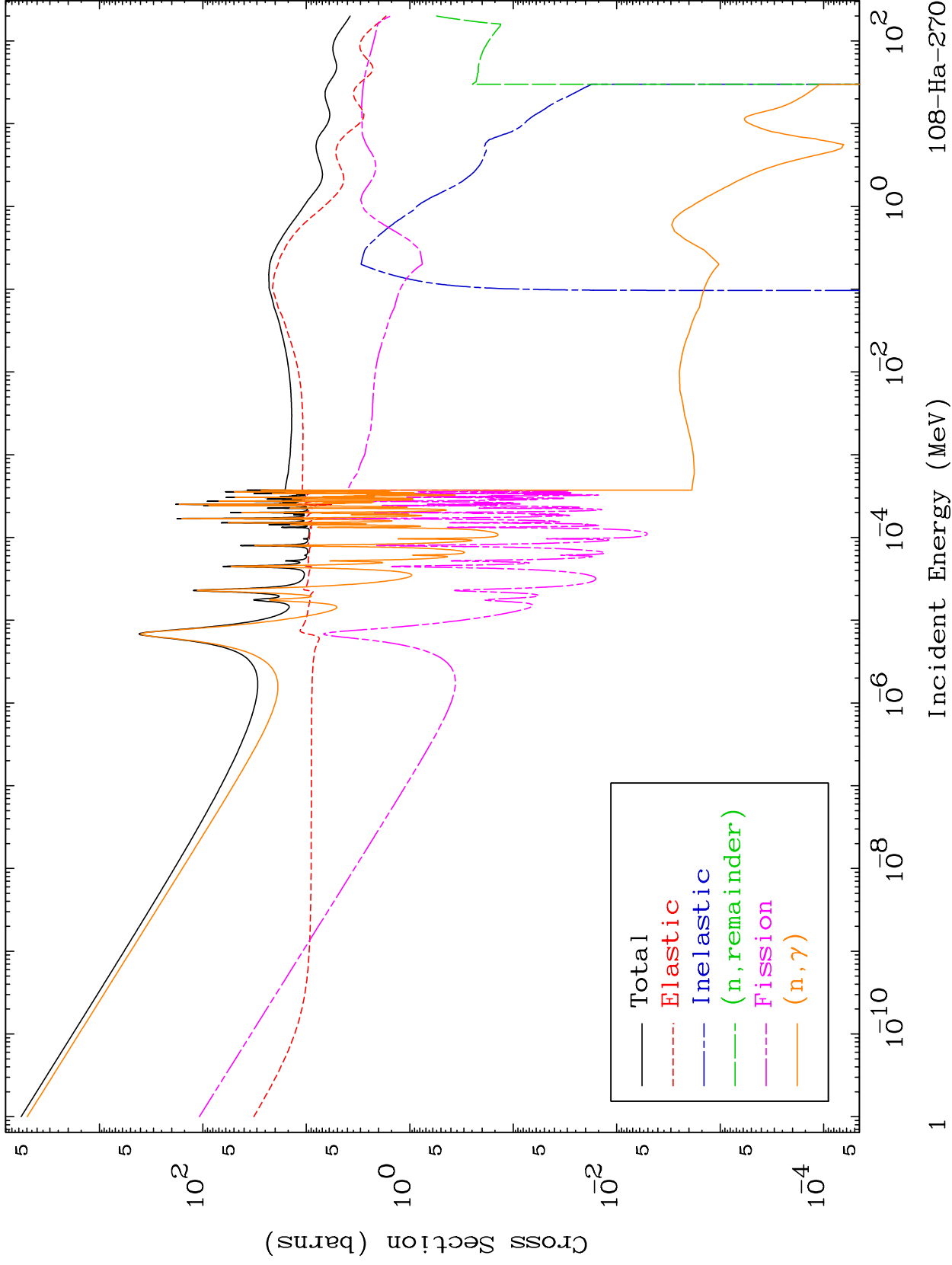
Press Mouse Button to Start

MAT 870

Major

293 Kelvin Cross Sections

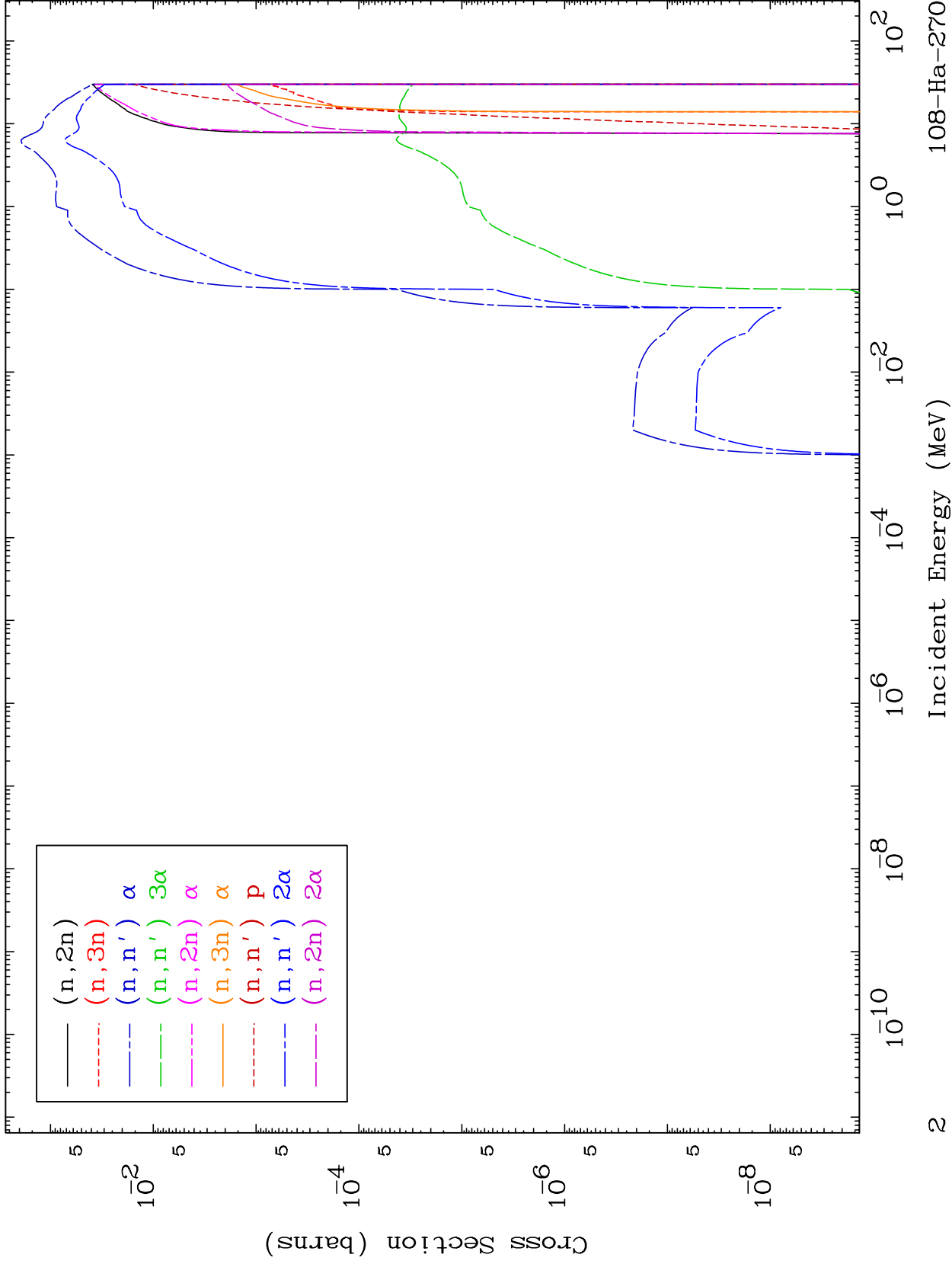
108-Ha-270



MAT 870

Neutron Production
293 Kelvin Cross Sections

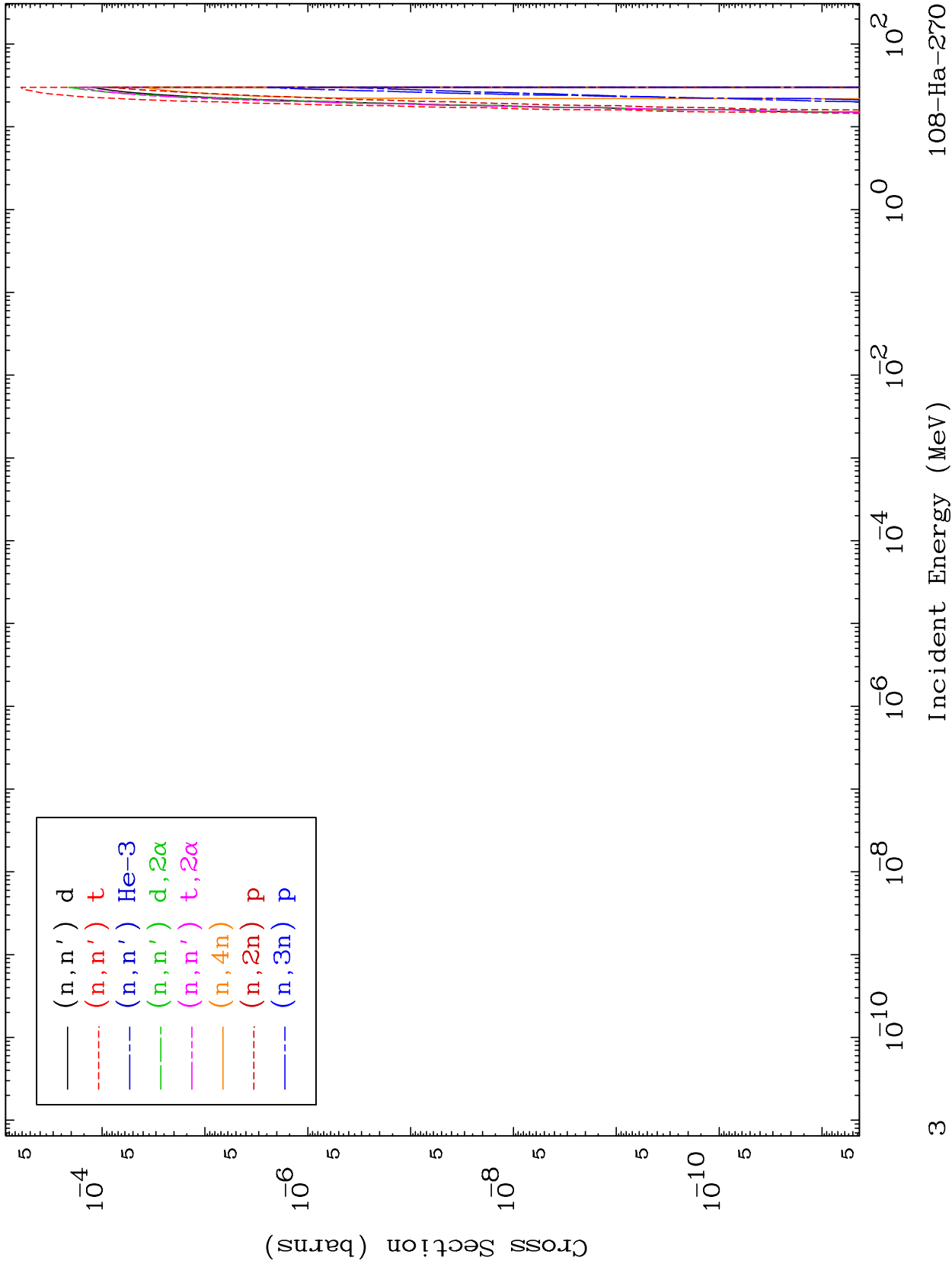
108-Ha-270



MAT 870

Neutron Production
293 Kelvin Cross Sections

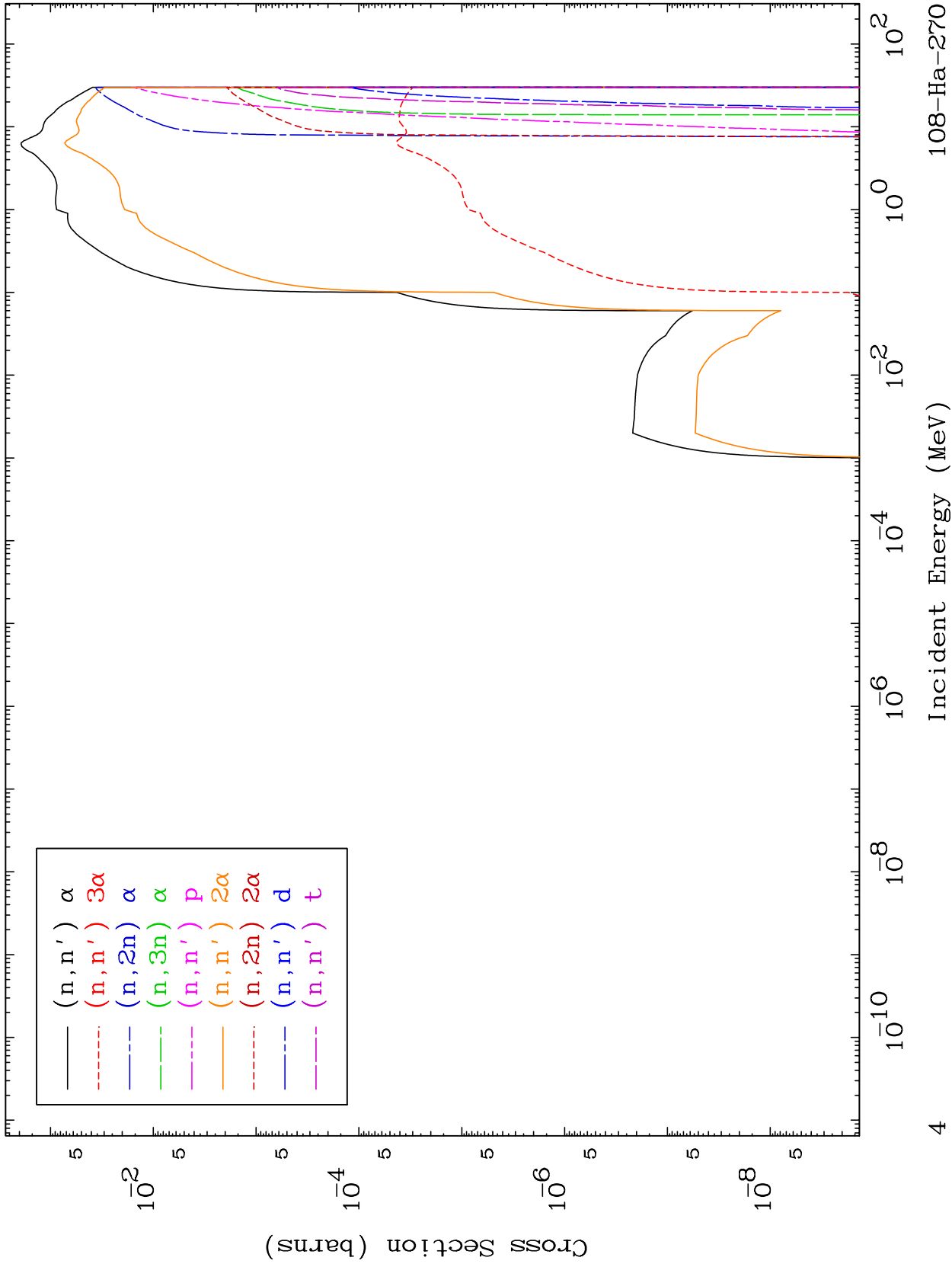
108-Ha-270

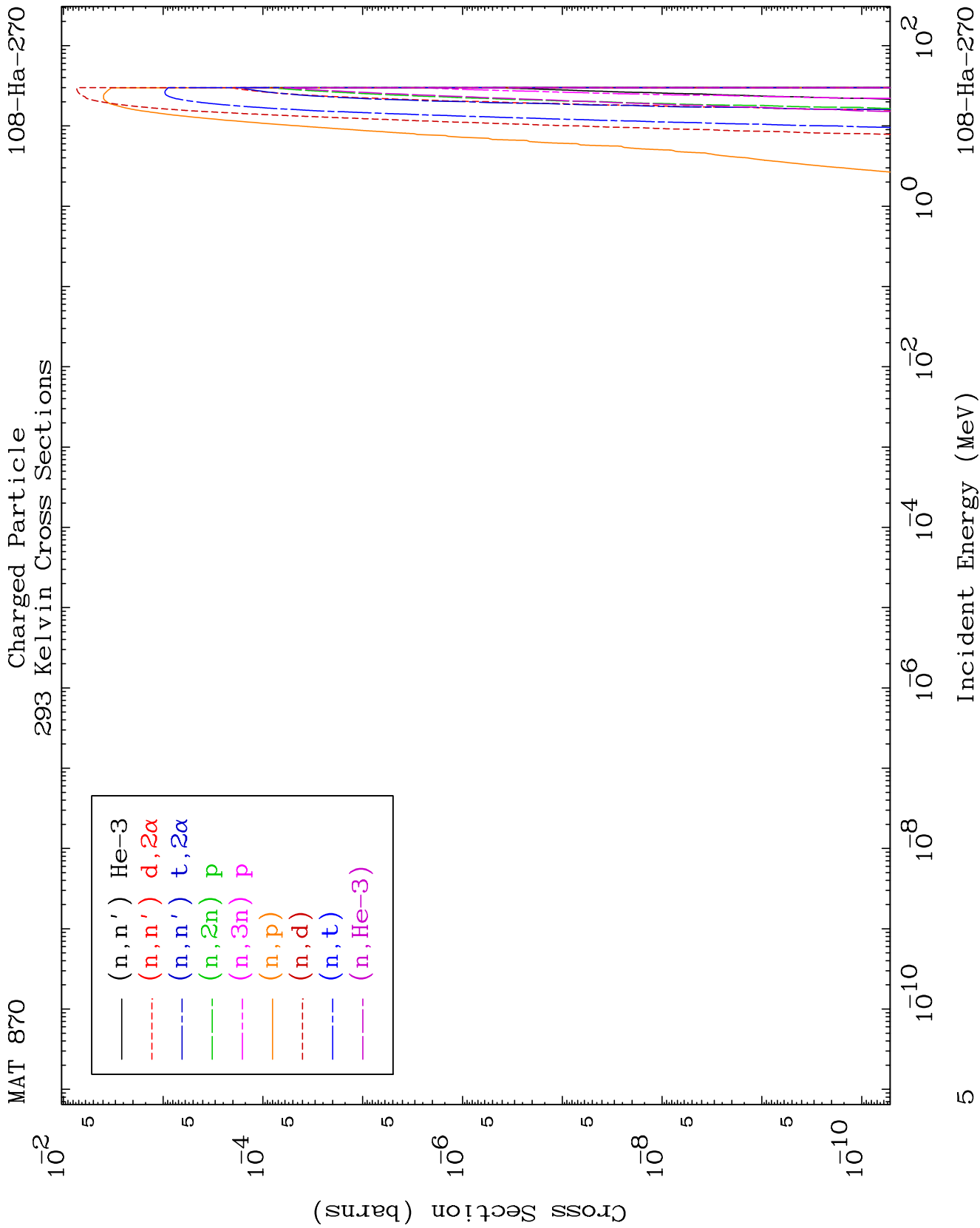


MAT 870

Charged Particle
293 Kelvin Cross Sections

108-Ha-270

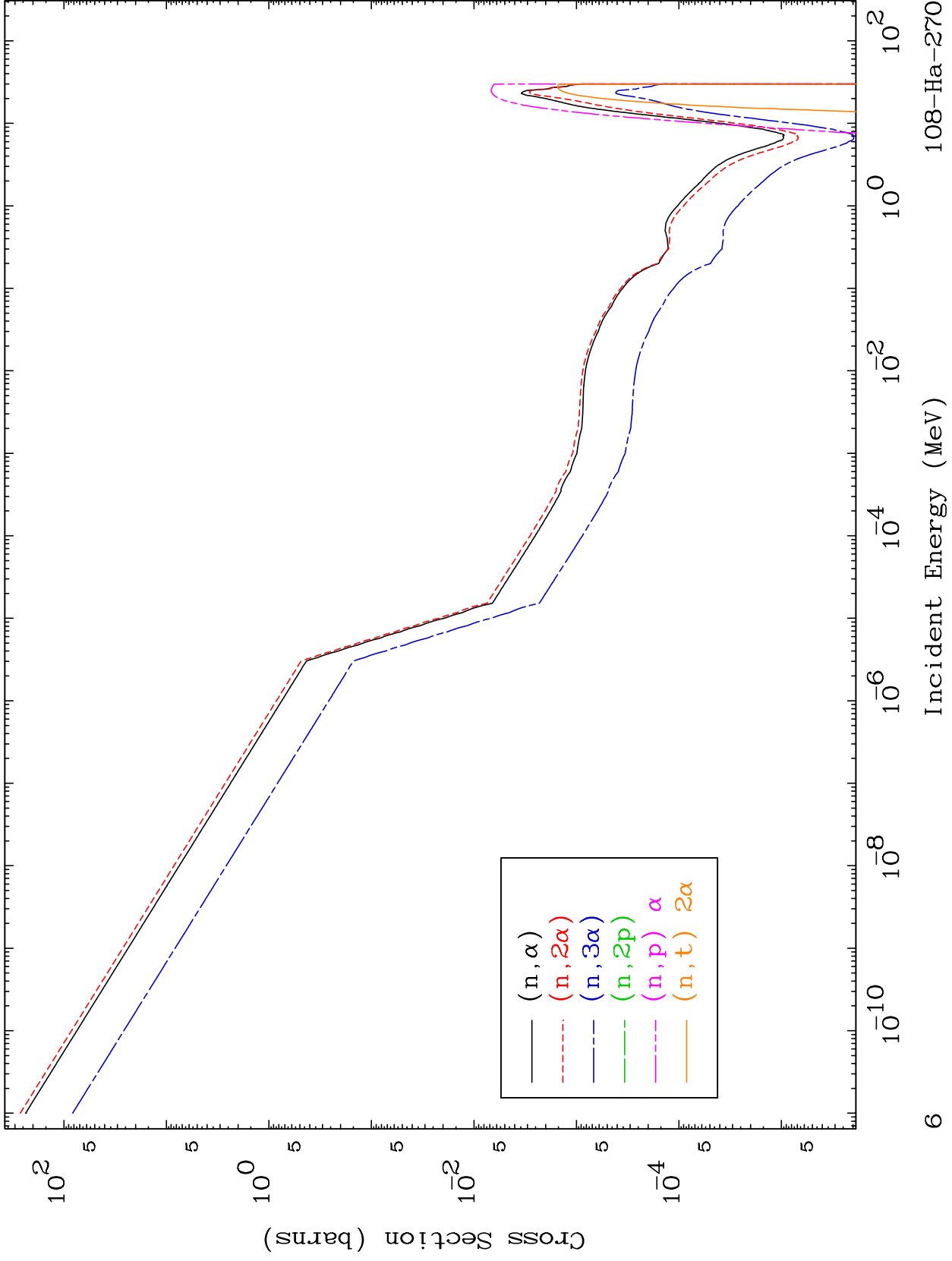




MAT 870

Charged Particle
293 Kelvin Cross Sections

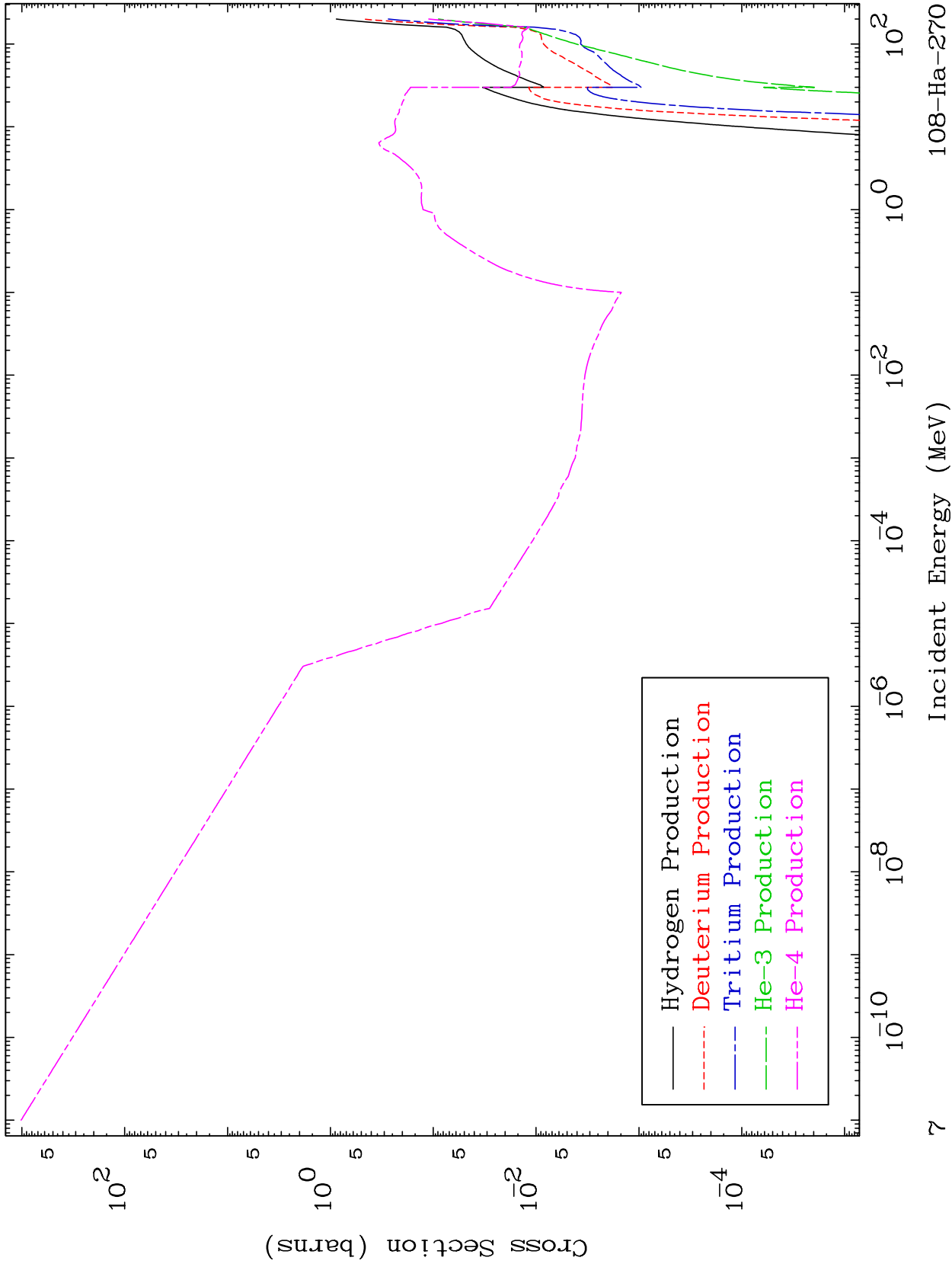
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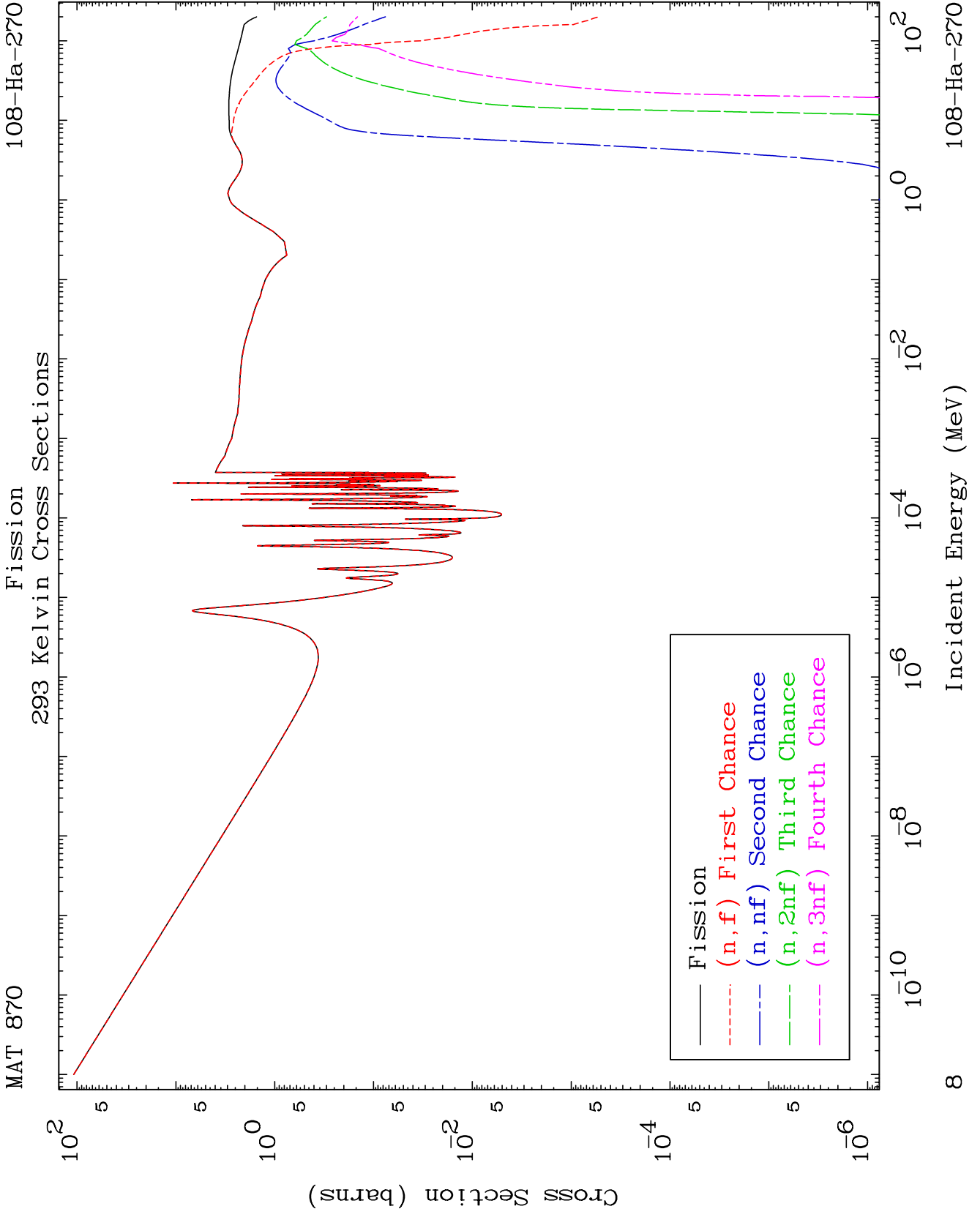


MAT 870

Particle Production
293 Kelvin Cross Sections

108-Ha-270

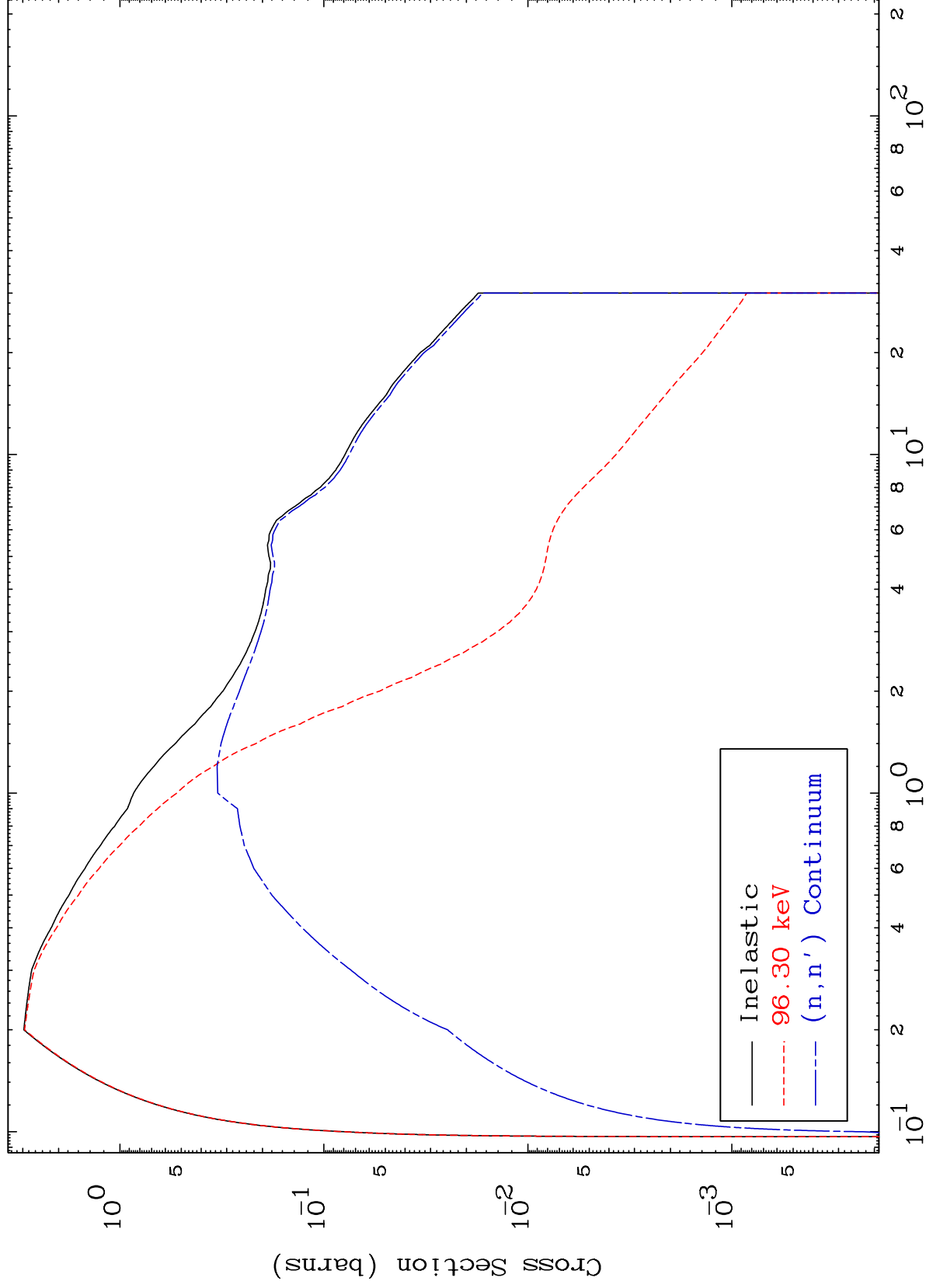




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(n,n') Level
293 Kelvin Cross Sections

108-Ha-270



Incident Energy (MeV)

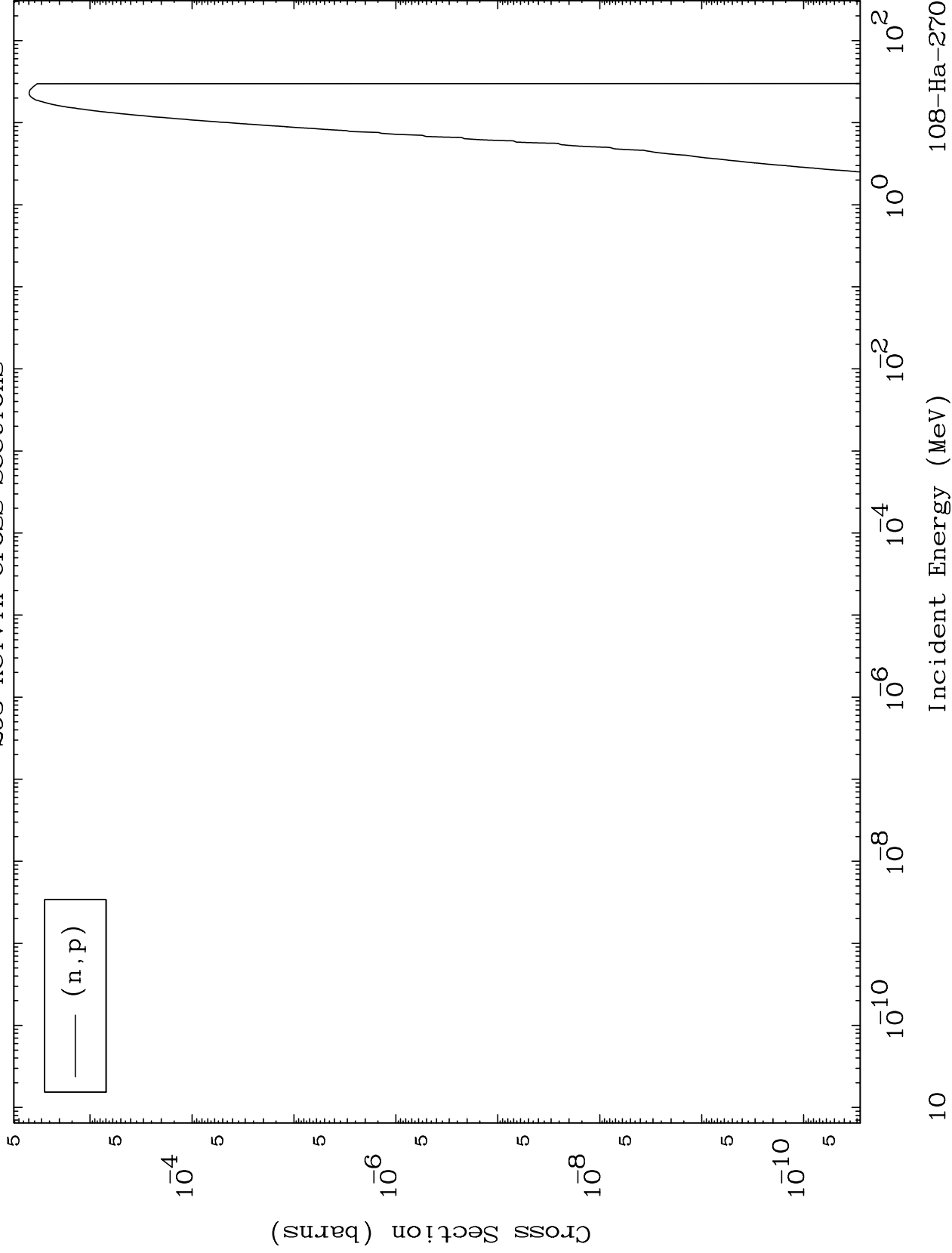
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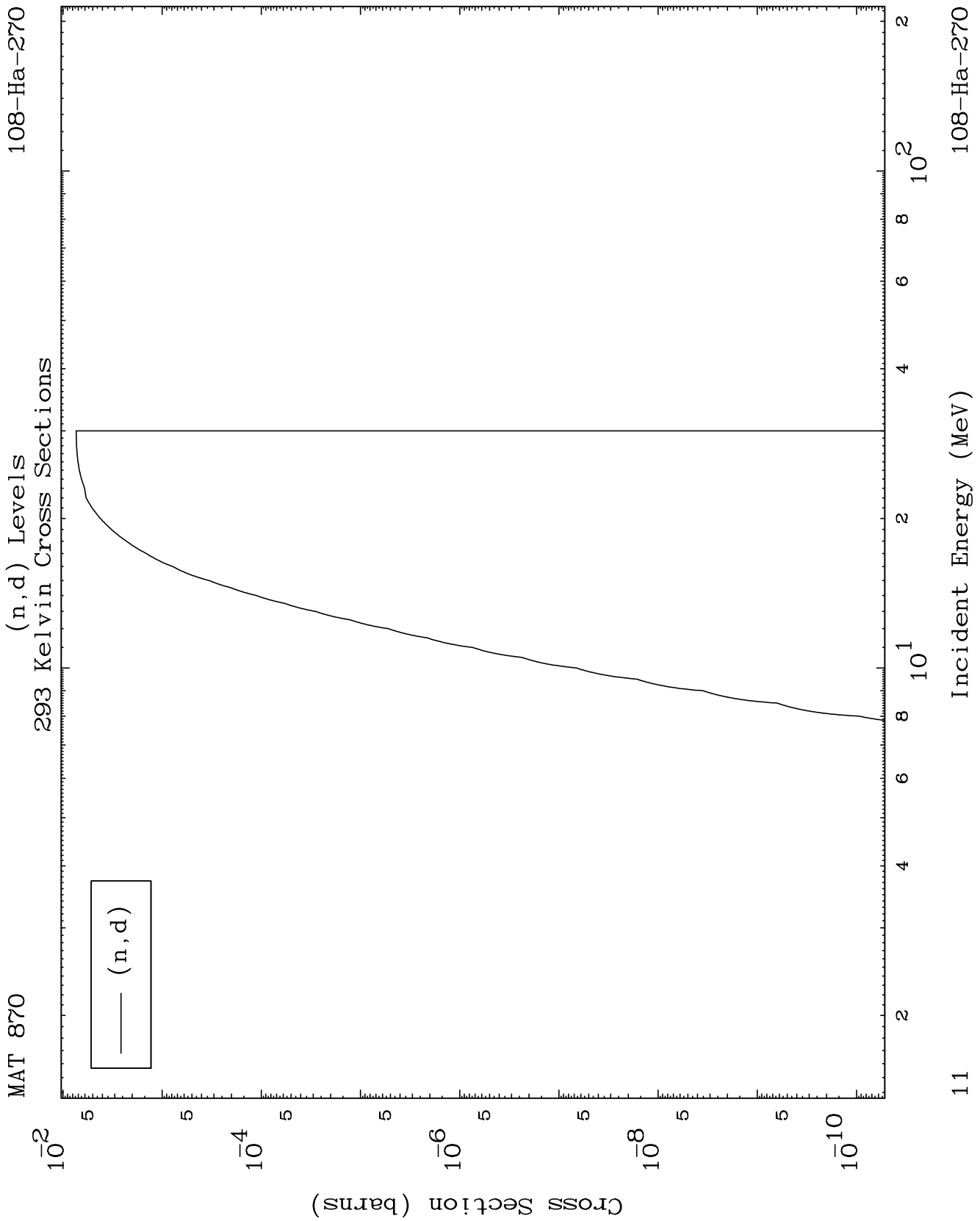
9

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(n,p) Levels
293 Kelvin Cross Sections

108-Ha-270

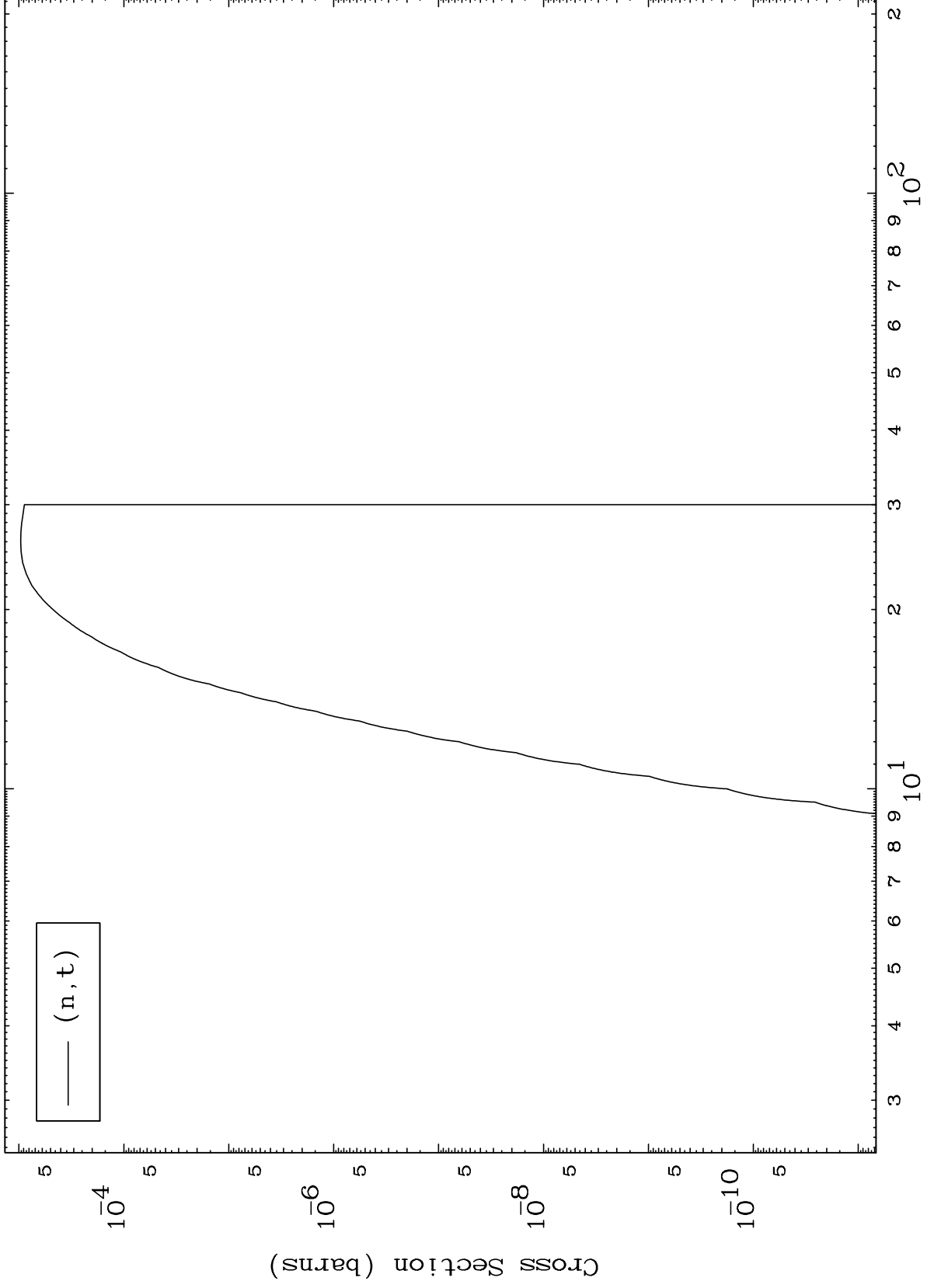




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(n,t) Levels
293 Kelvin Cross Sections

108-Ha-270



12

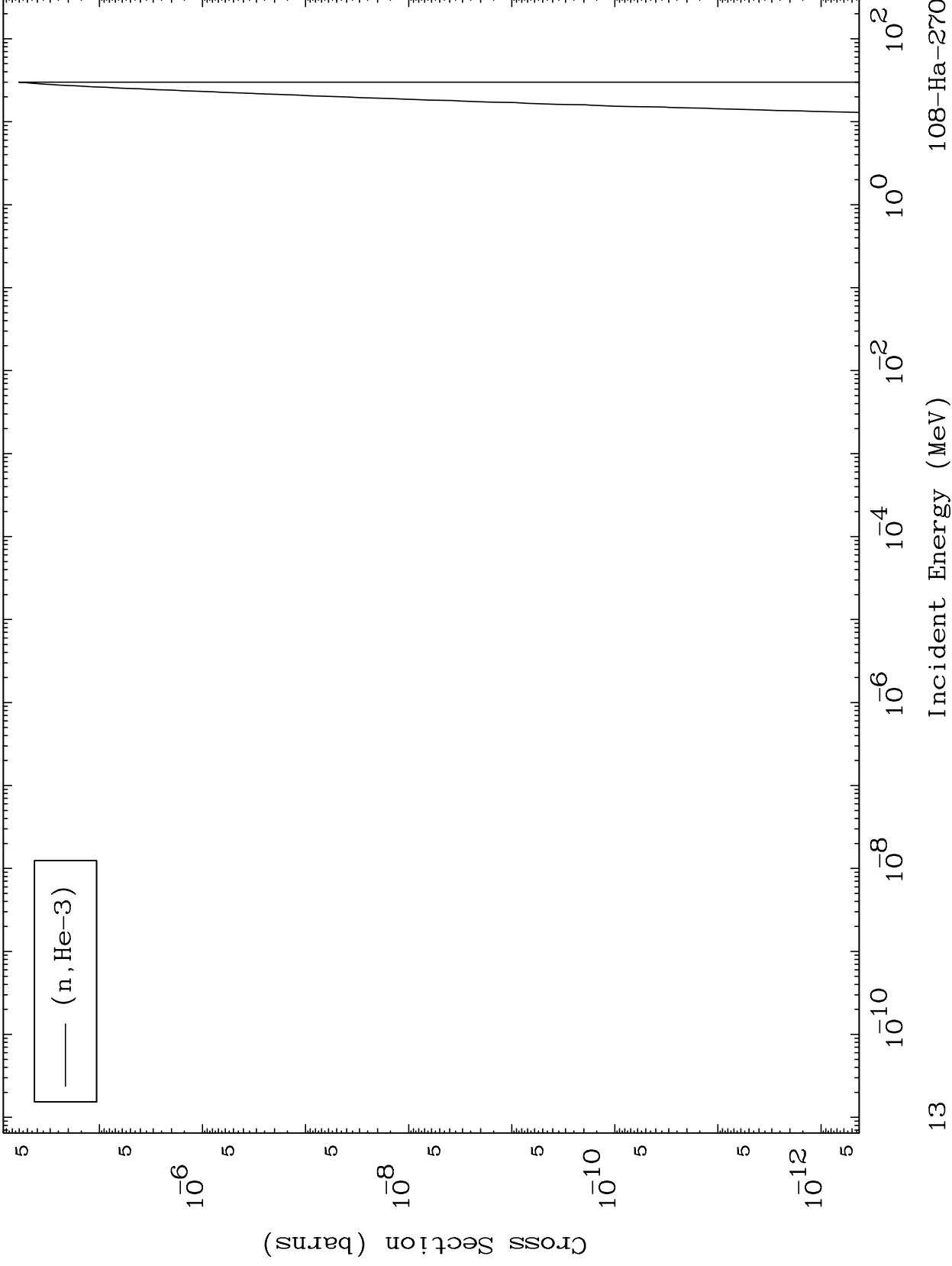
Incident Energy (MeV)

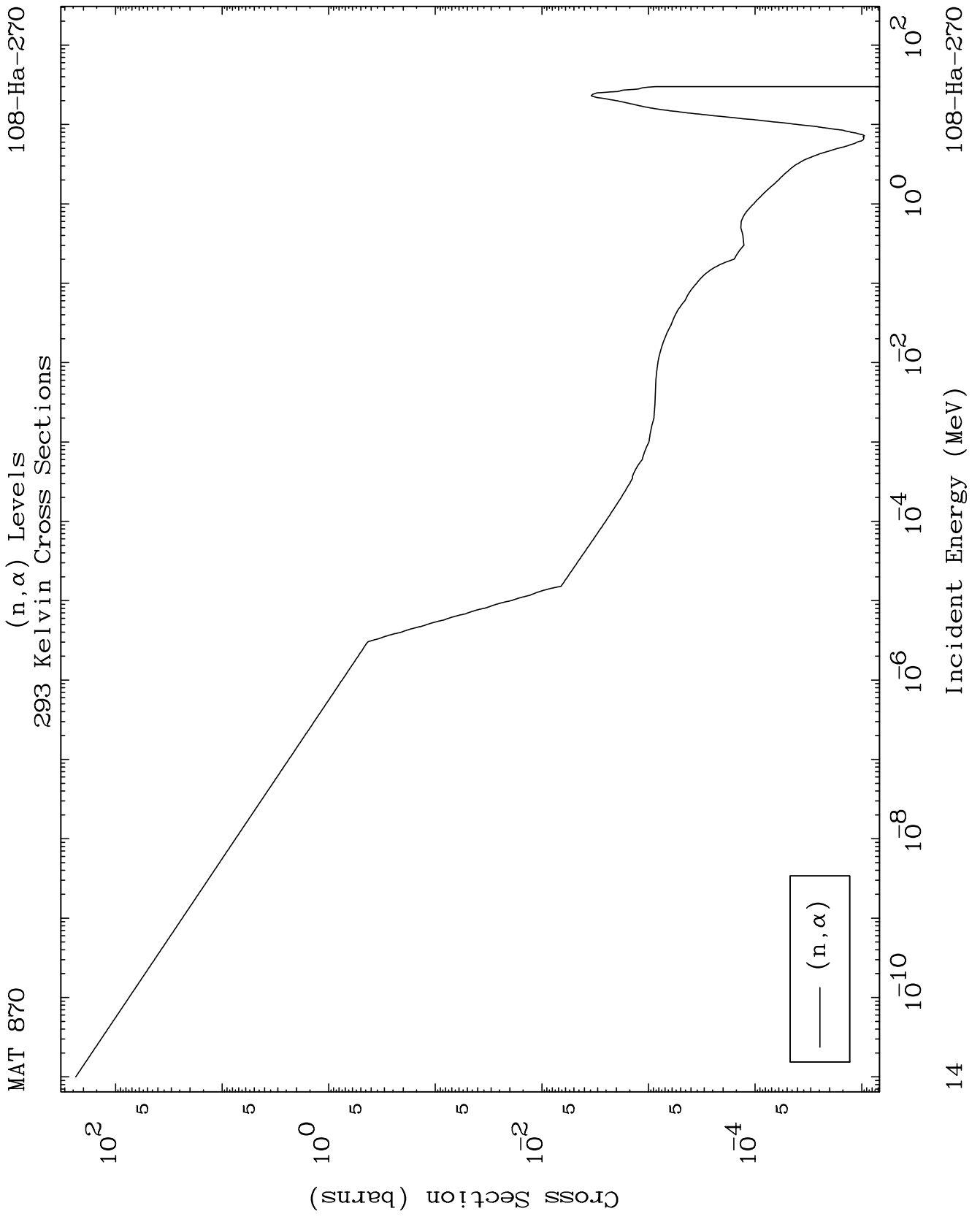
108-Ha-270

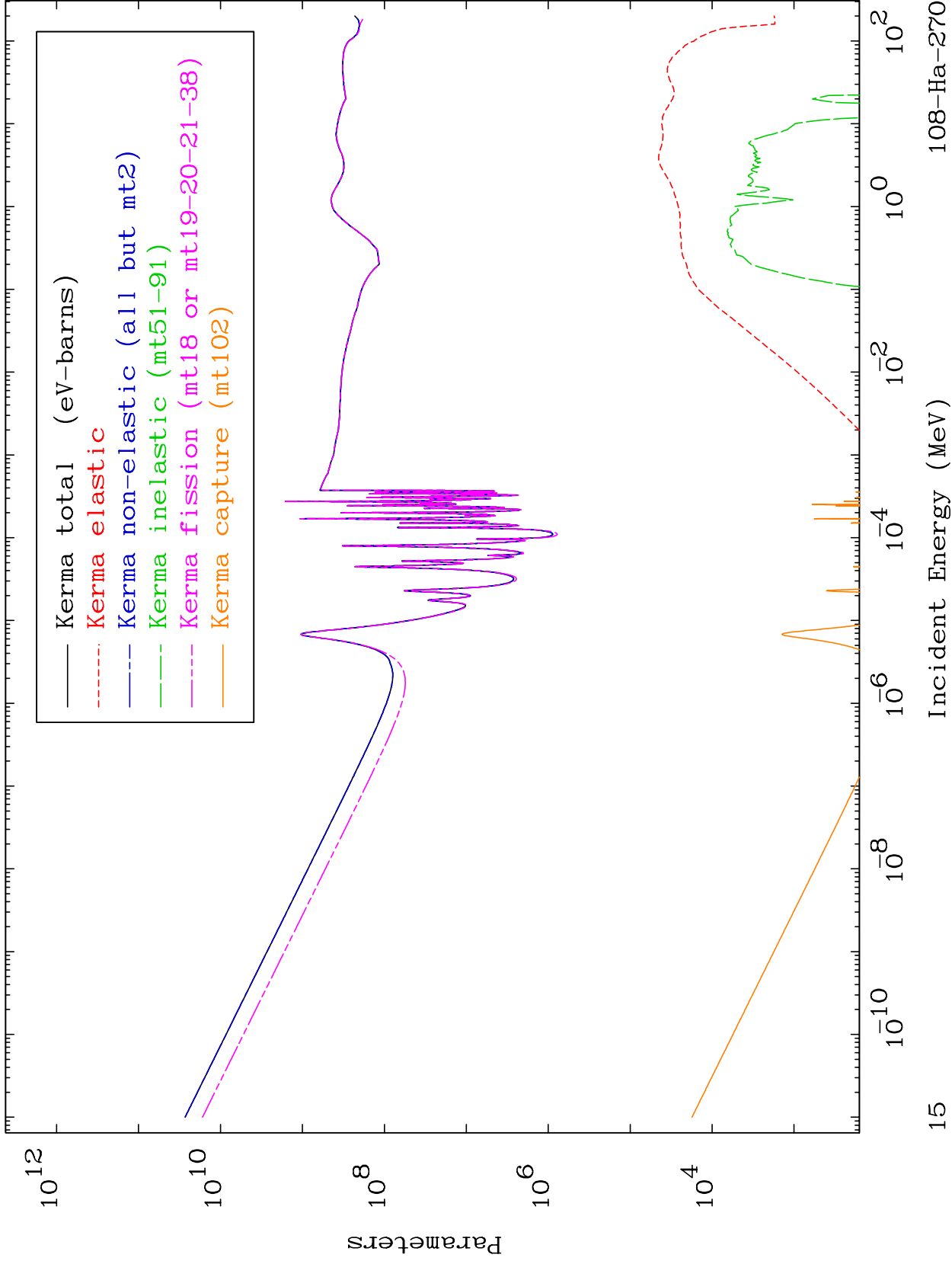
MAT 870

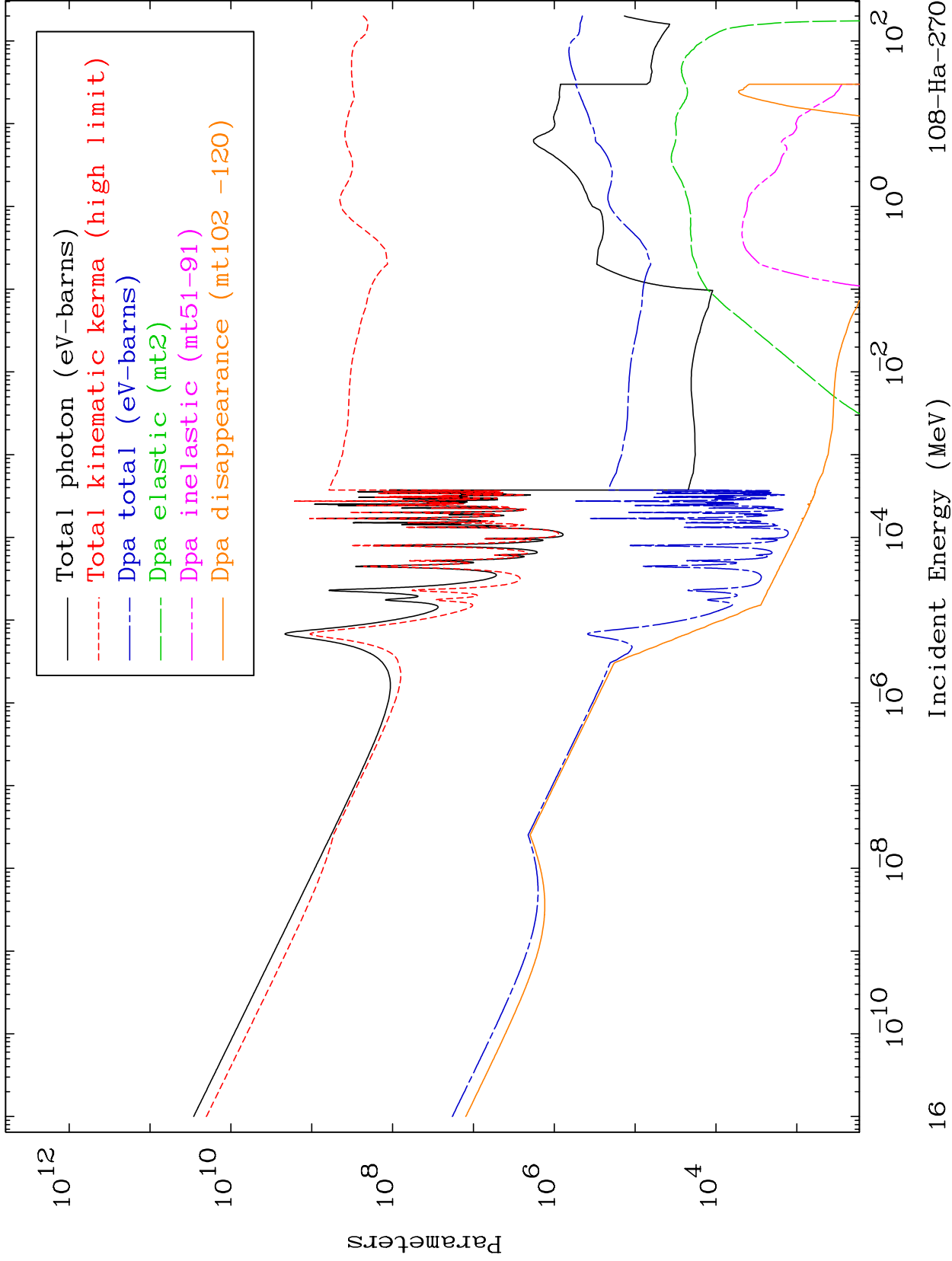
(n,He3) Levels
293 Kelvin Cross Sections

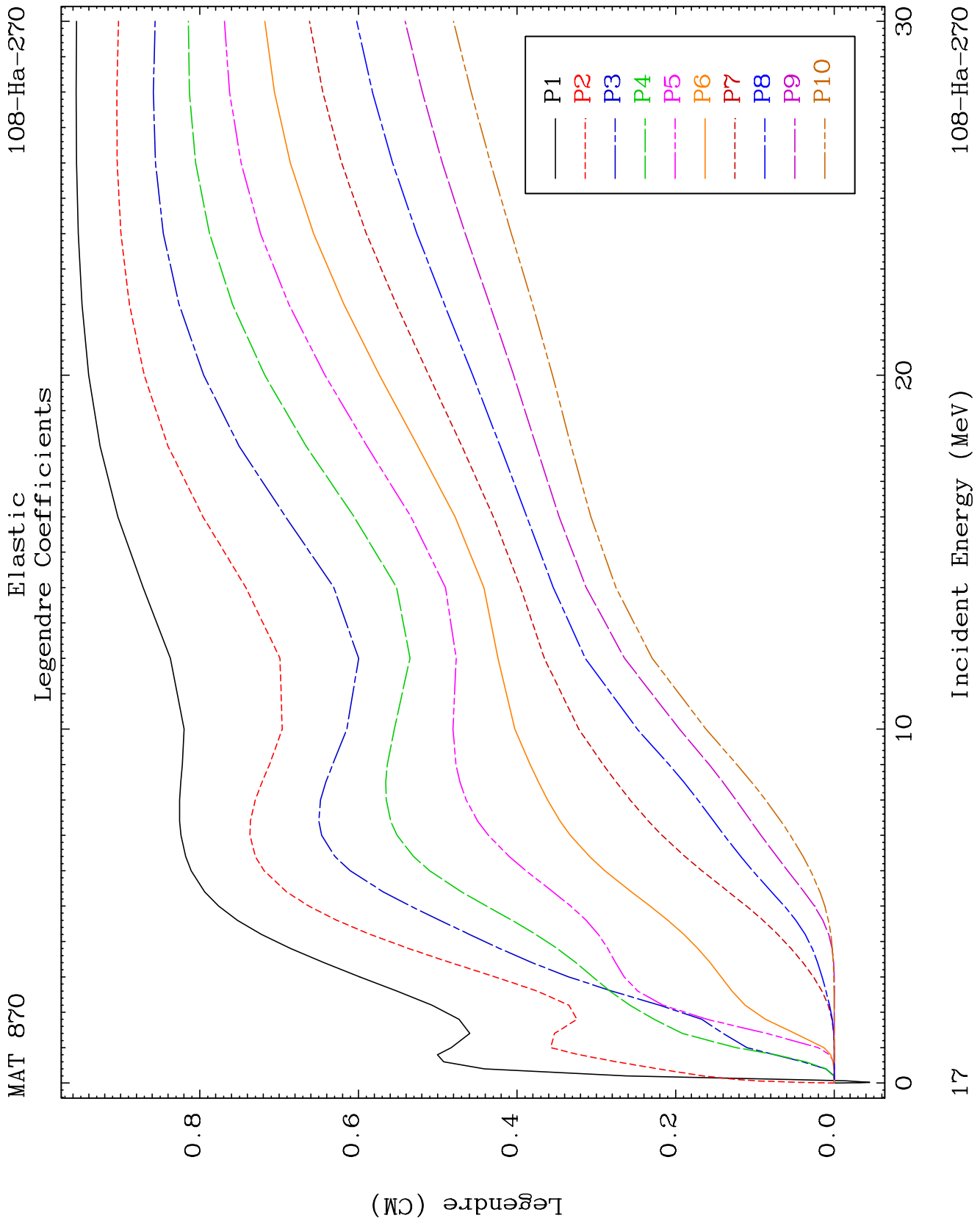
108-Ha-270

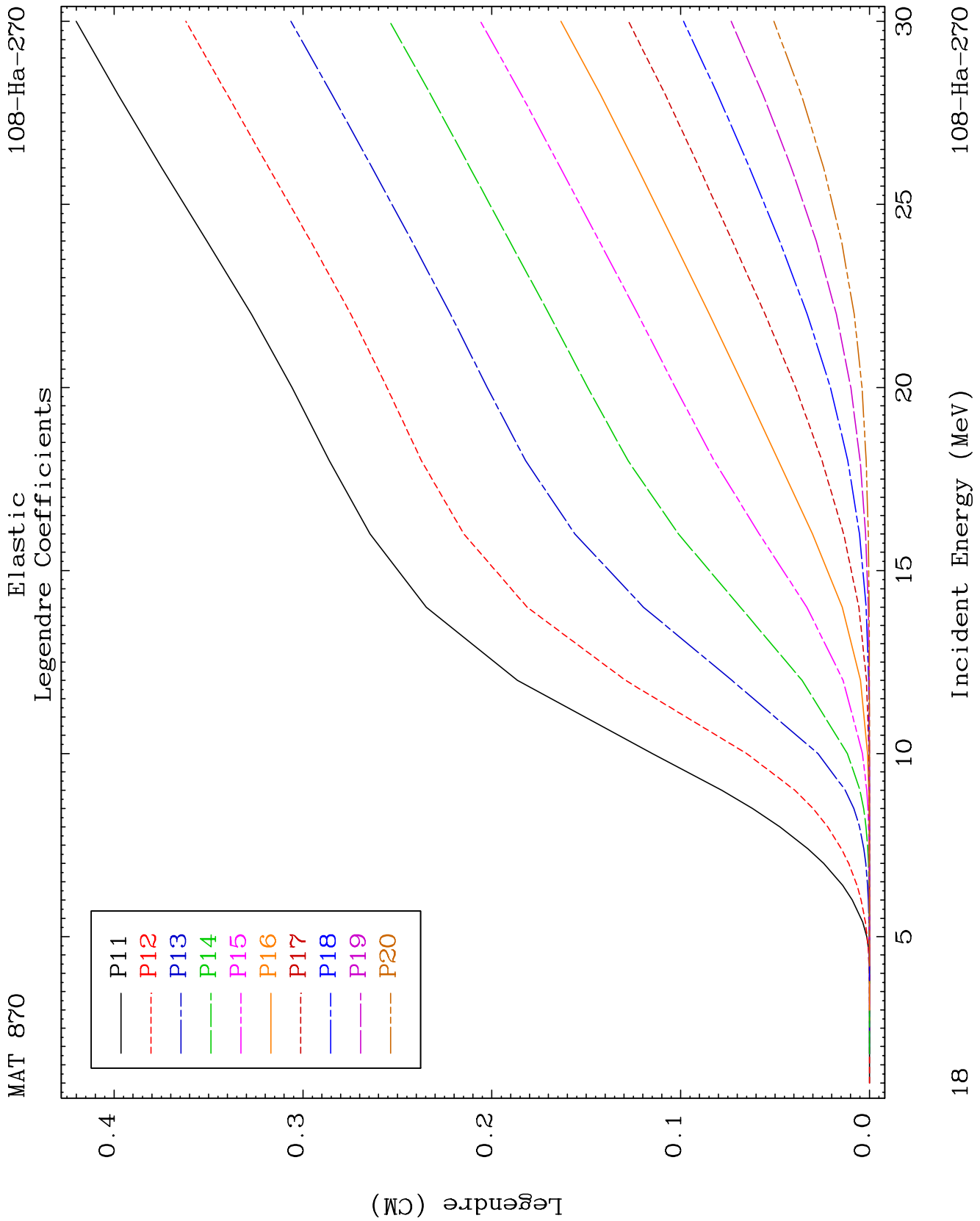


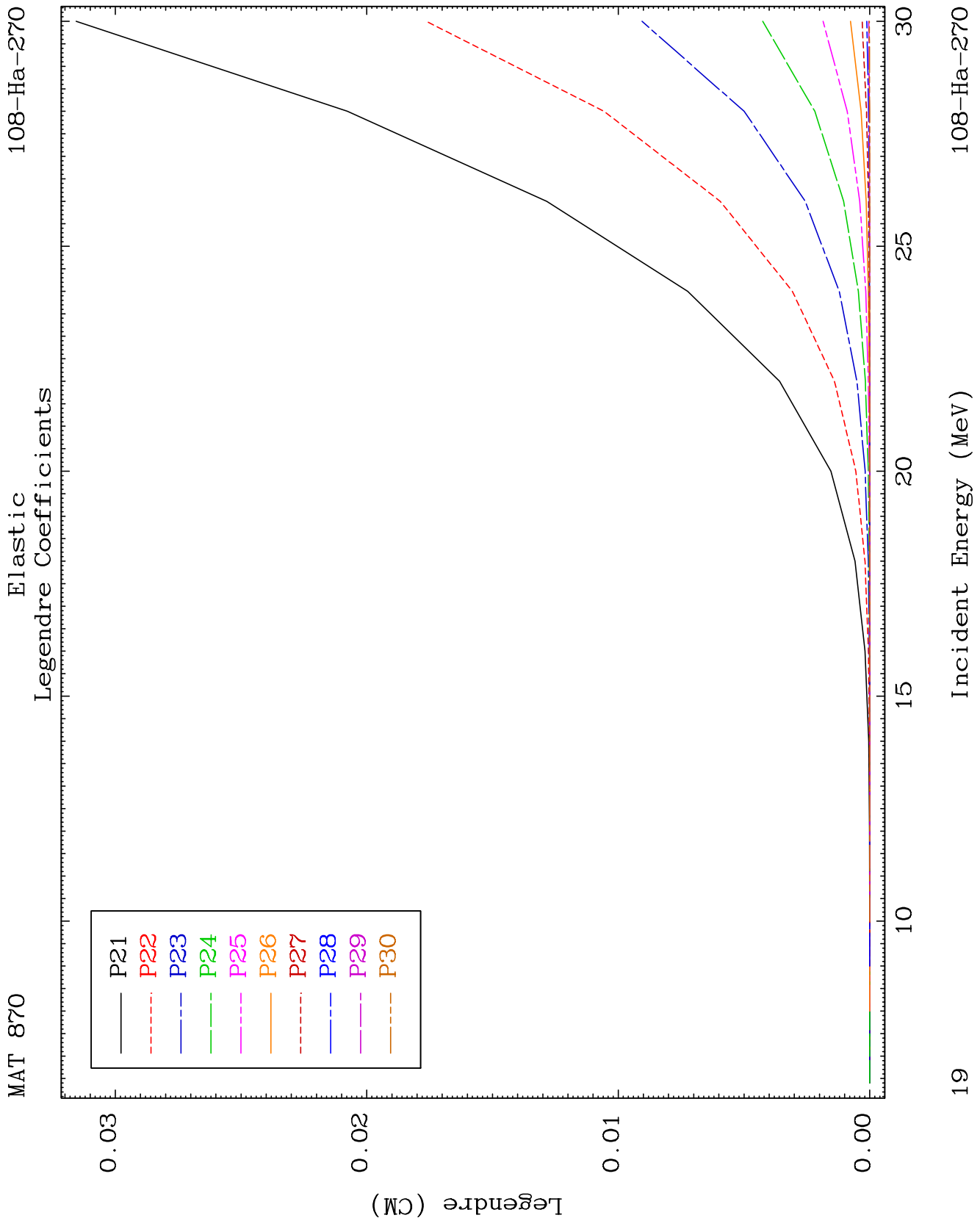












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Elastic Legendre Coefficients

108-Ha-270

19

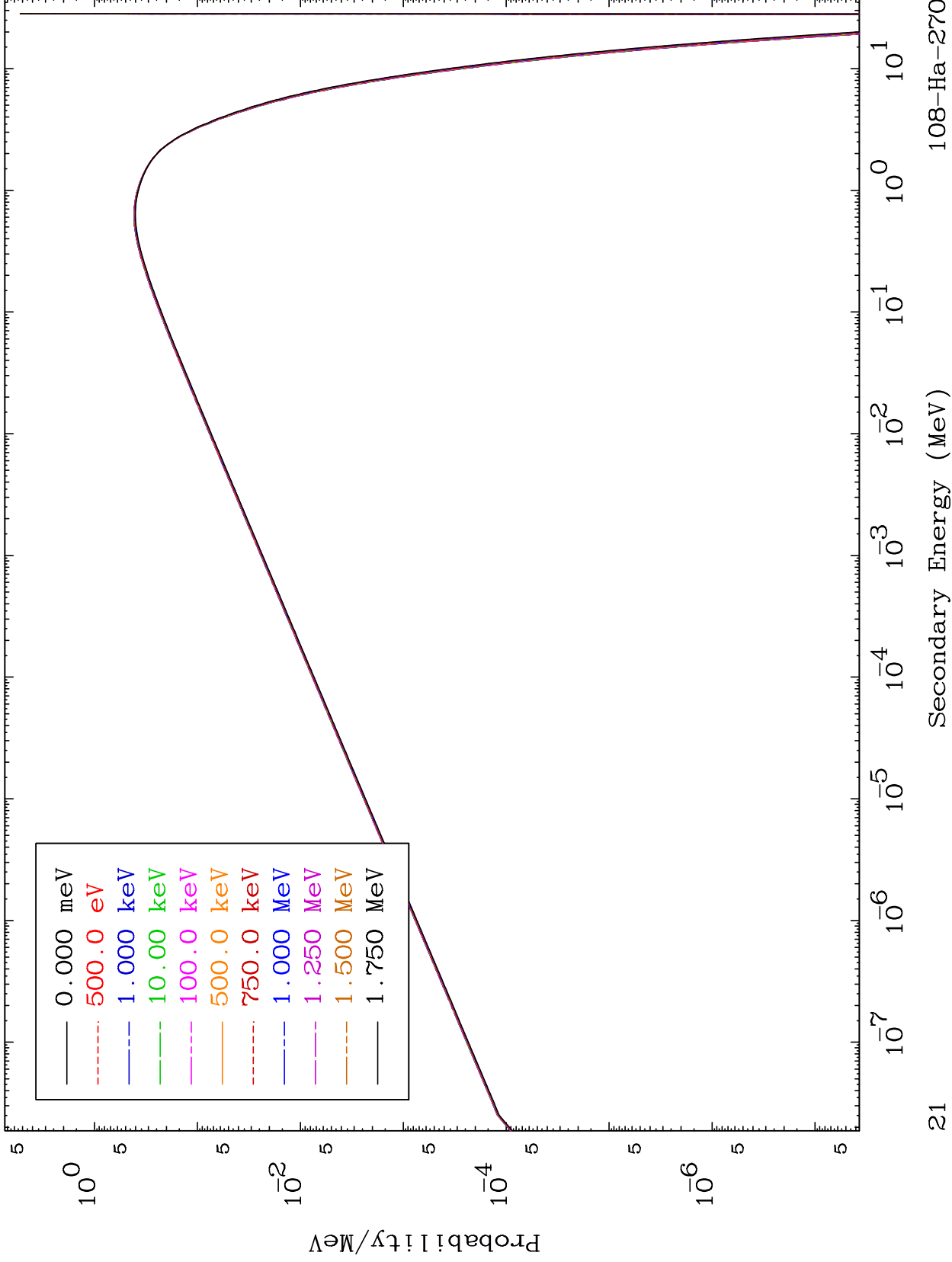
Incident Energy (MeV)

108-Ha-270

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Fission Energy Distribution

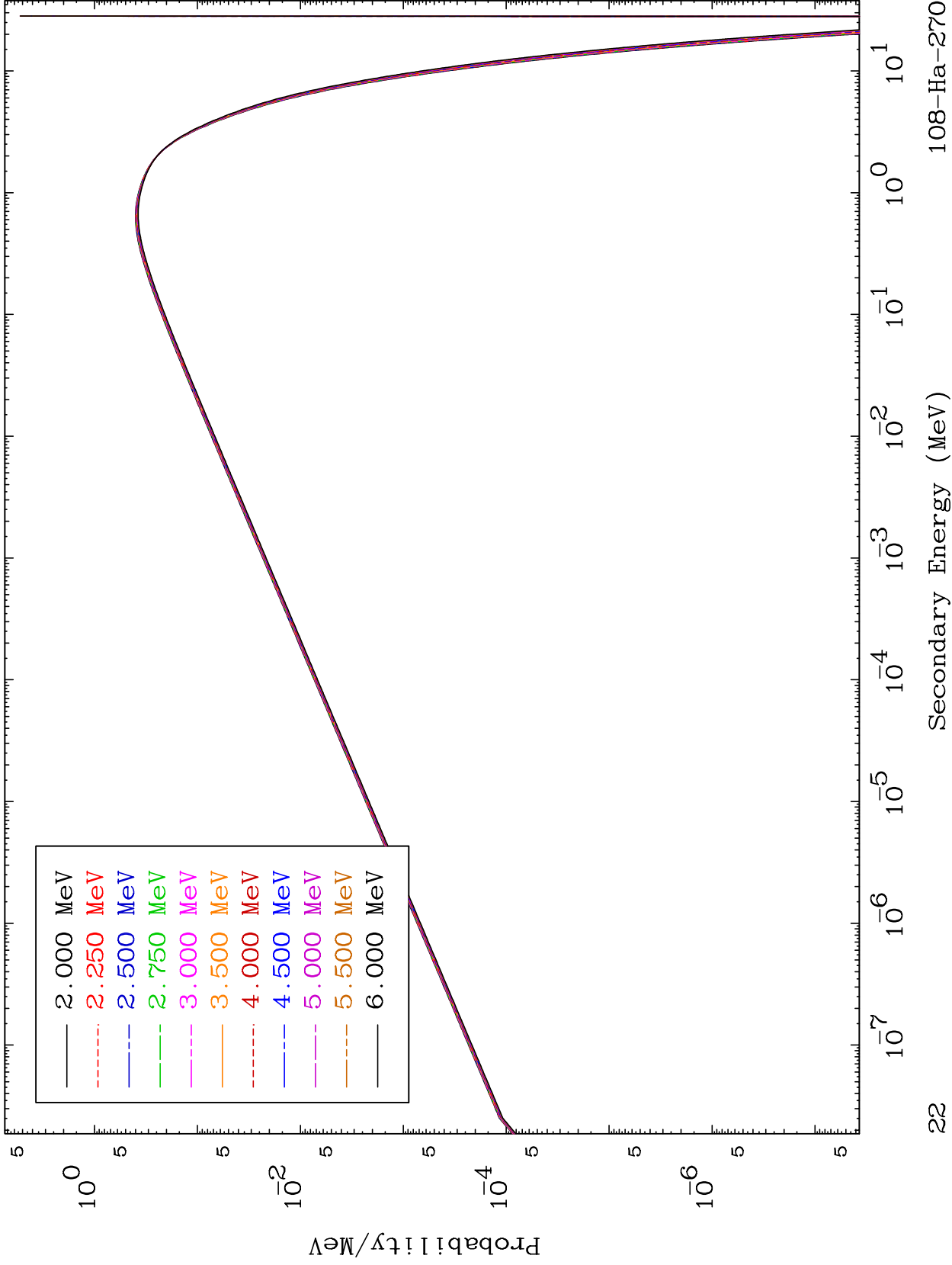
108-Ha-270



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Fission Energy Distribution

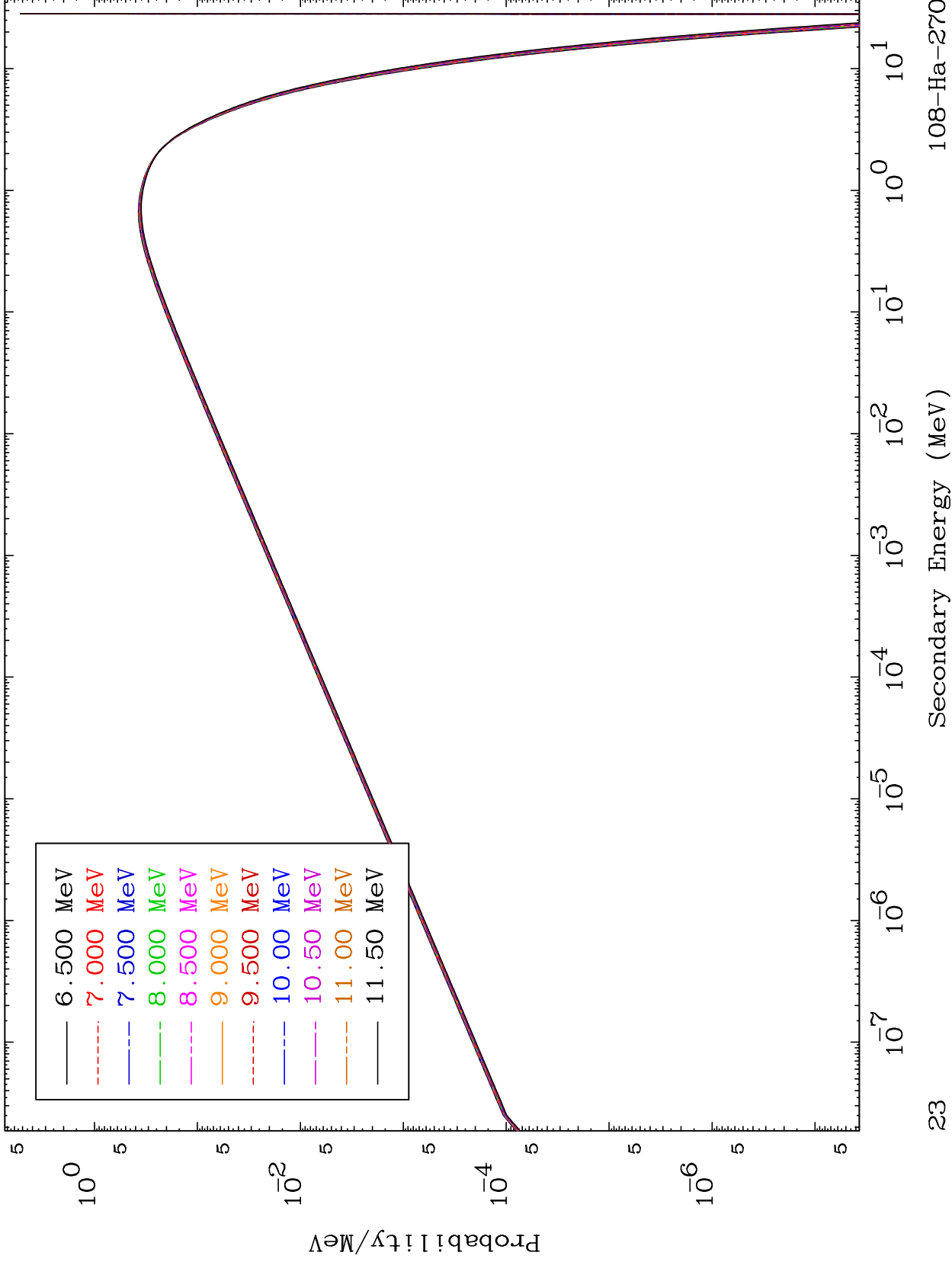
108-Ha-270



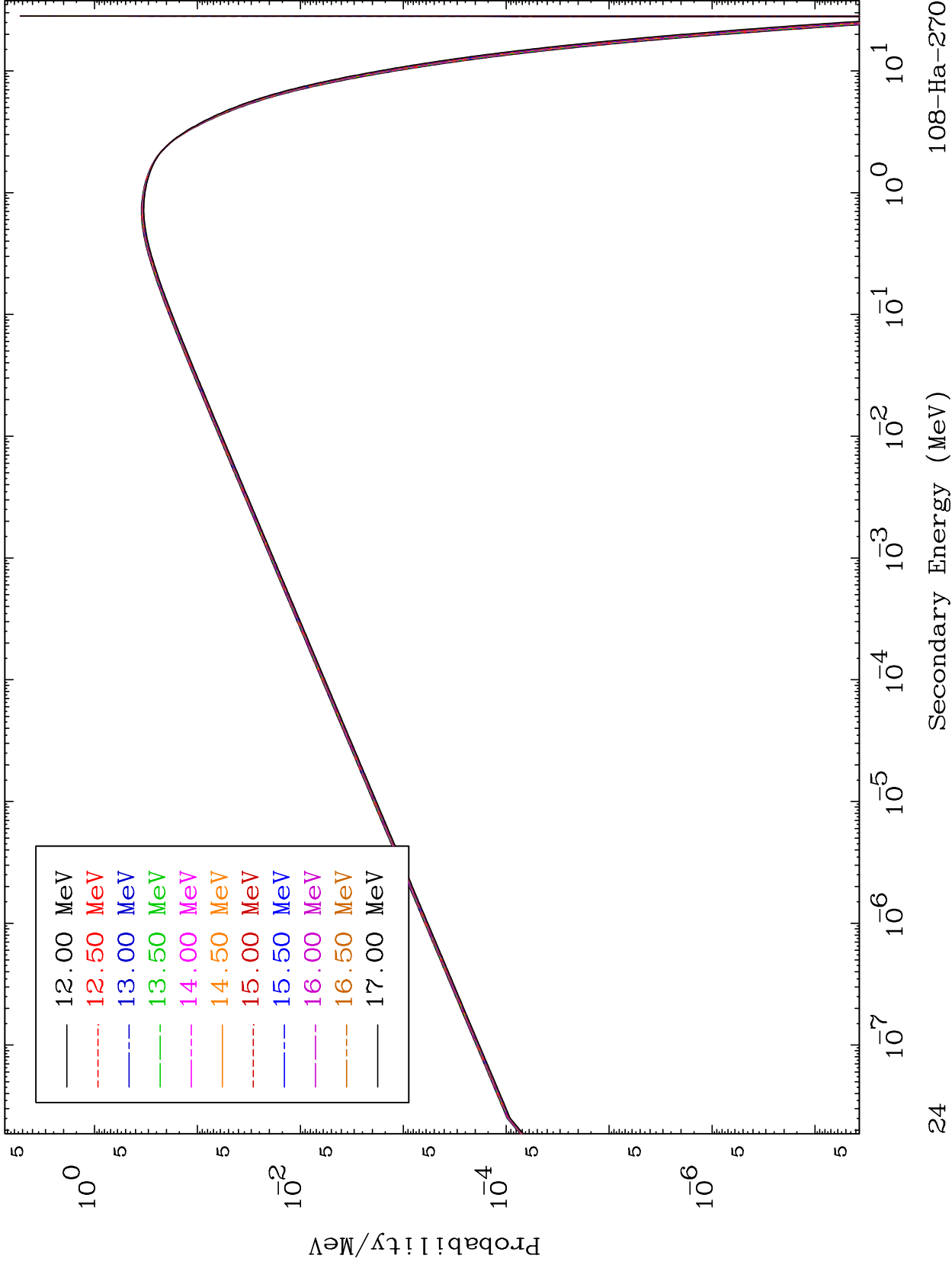
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Fission Energy Distribution

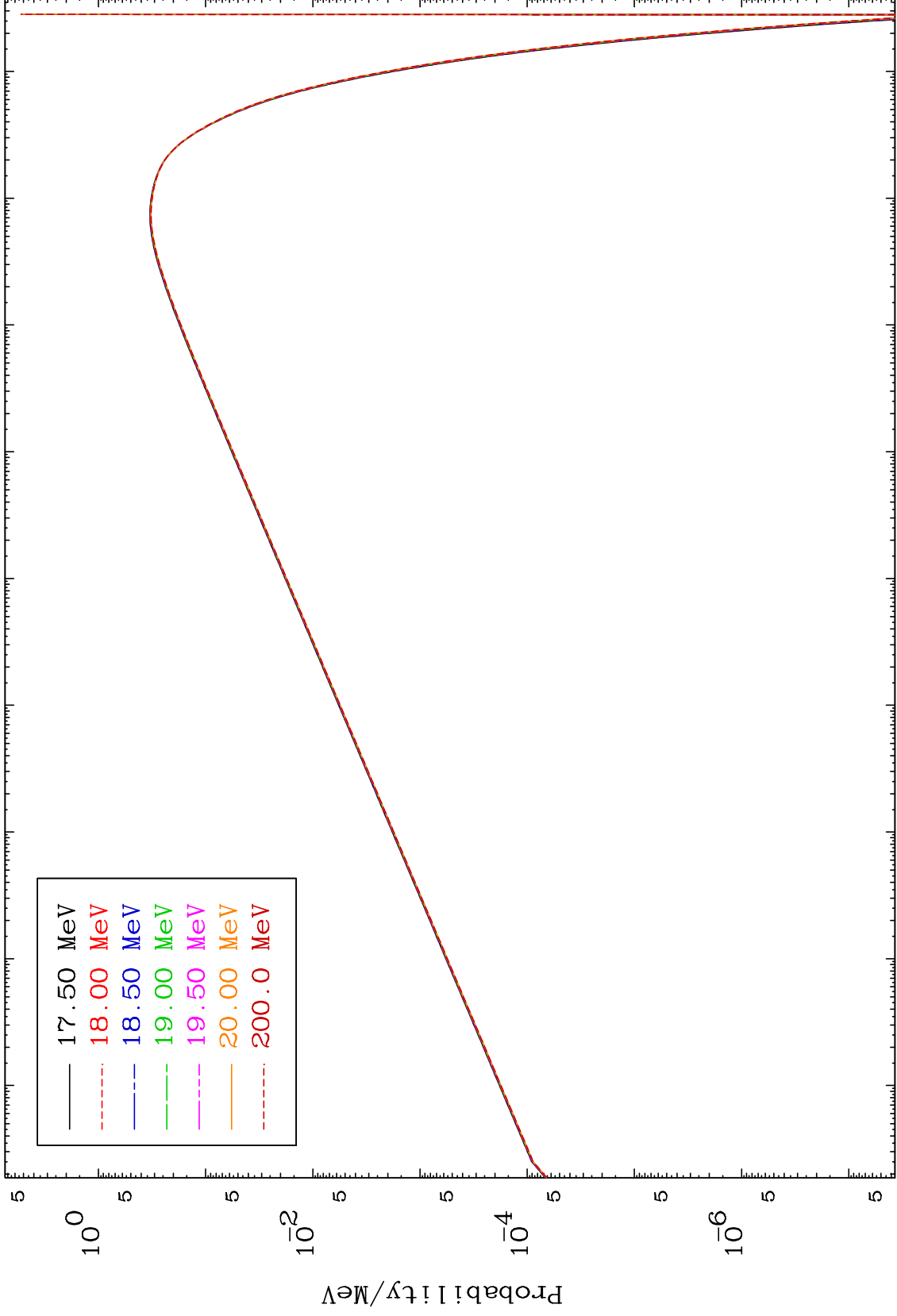
108-Ha-270



Fission
Energy Distribution



Fission
Energy Distribution

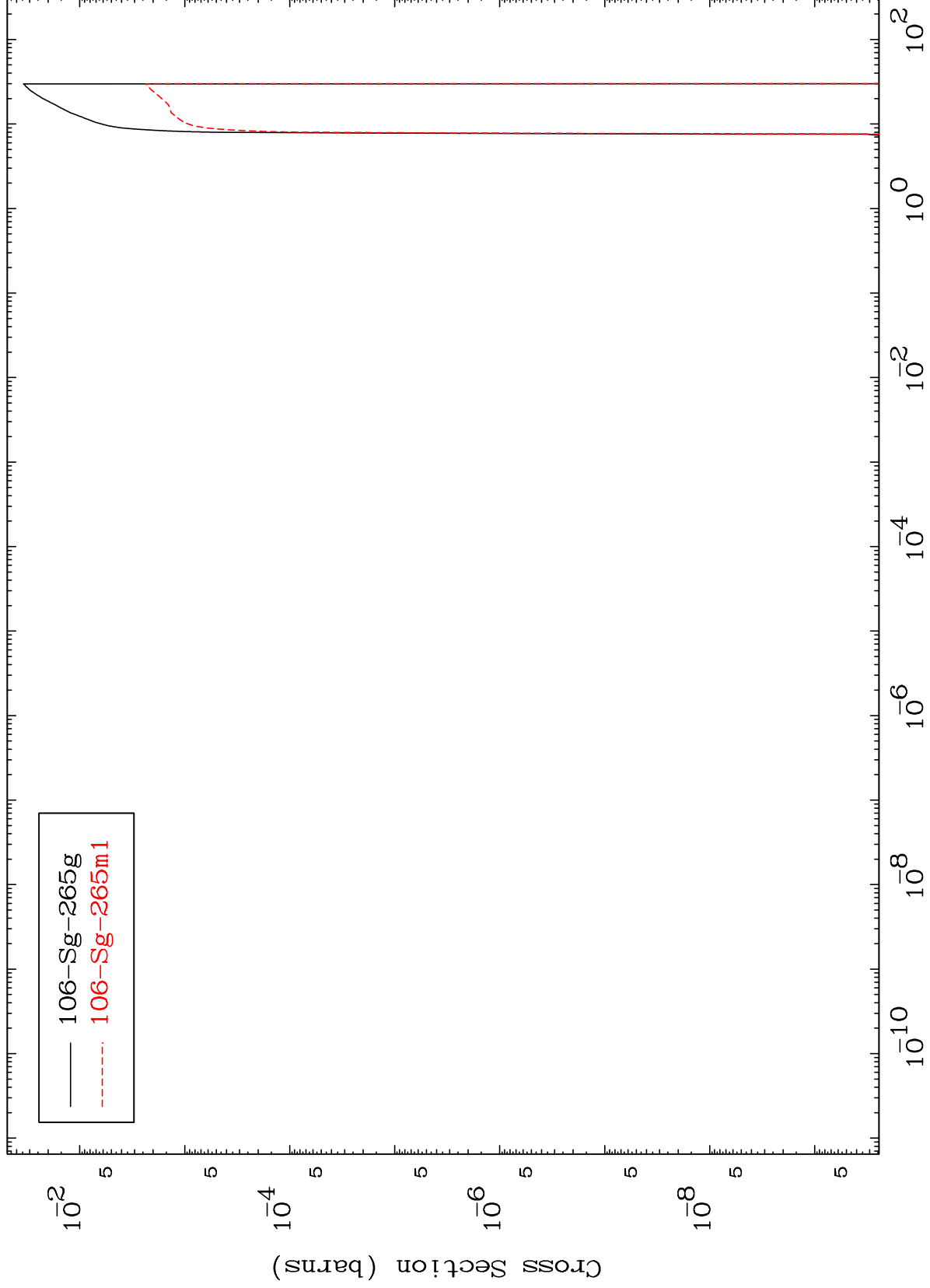


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(n,2n) α

108-Ha-270

Radionuclide Production Cross Section



26

Incident Energy (MeV)

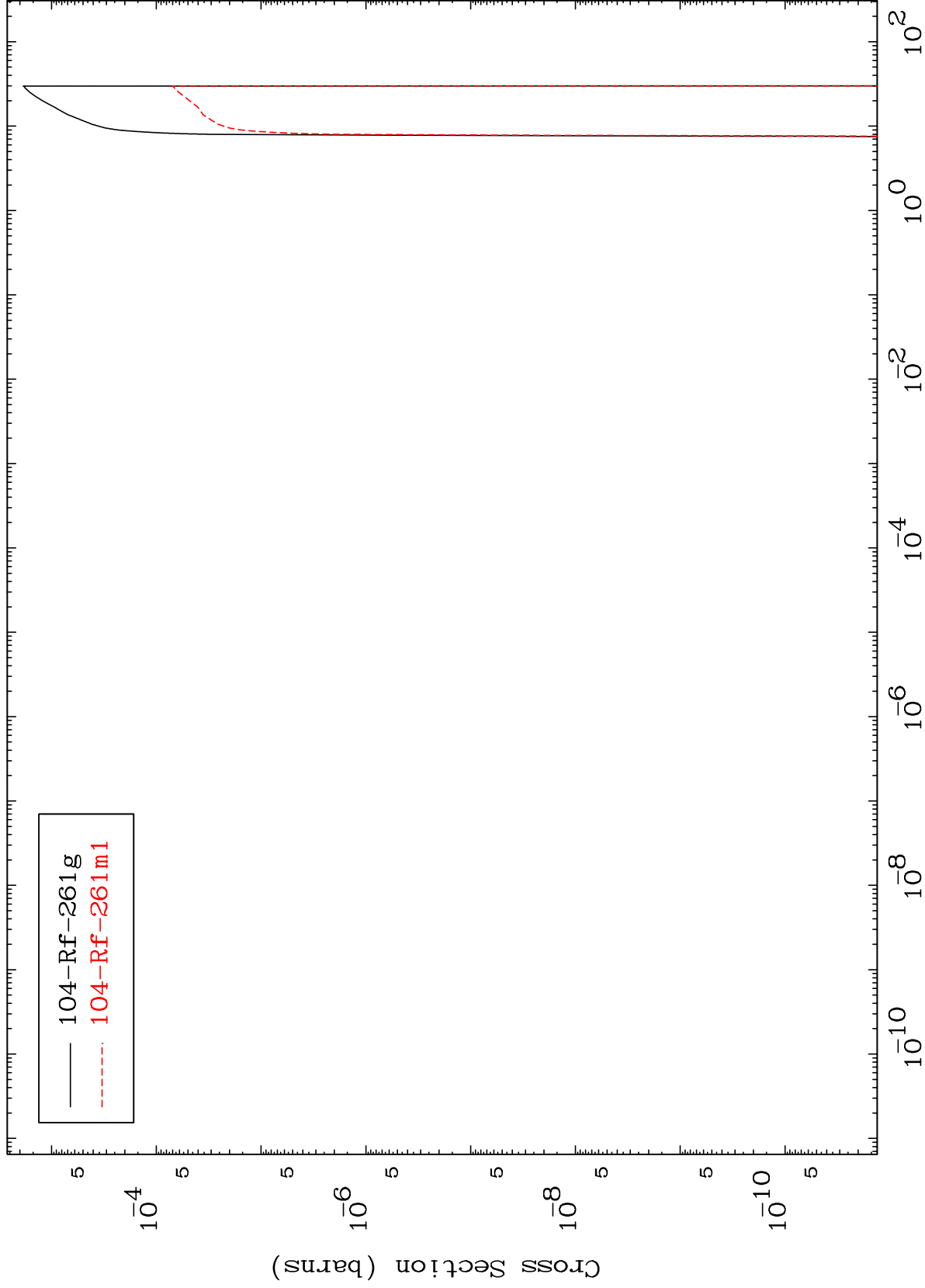
108-Ha-270

MAT 870

(n,2n) 2 α

108-Ha-270

Radionuclide Production Cross Section



27

Incident Energy (MeV)

108-Ha-270