

Program EVALPLOT  
(Version 2015-2)

by

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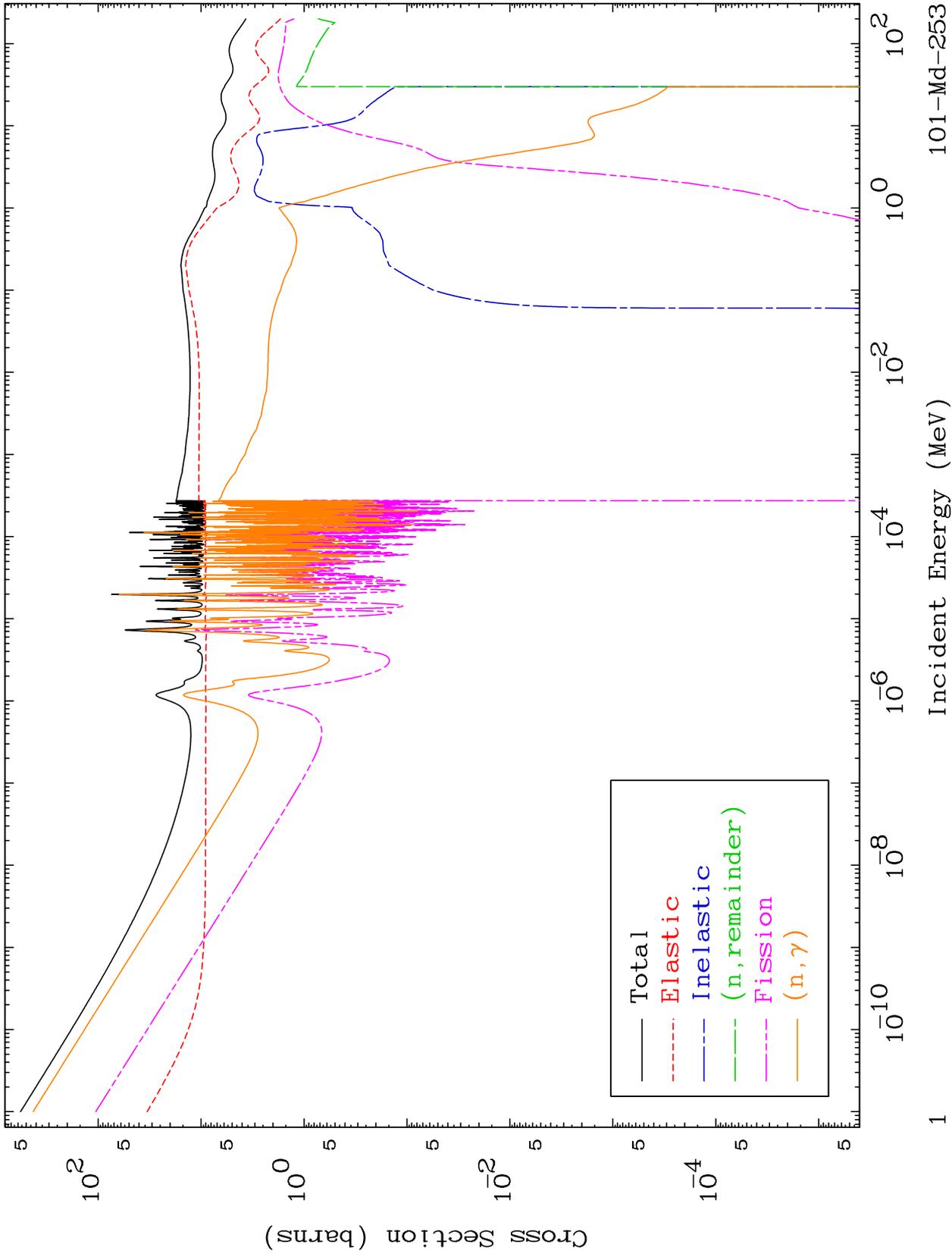
Press Mouse Button to Start

MAT 9954

Major

293 Kelvin Cross Sections

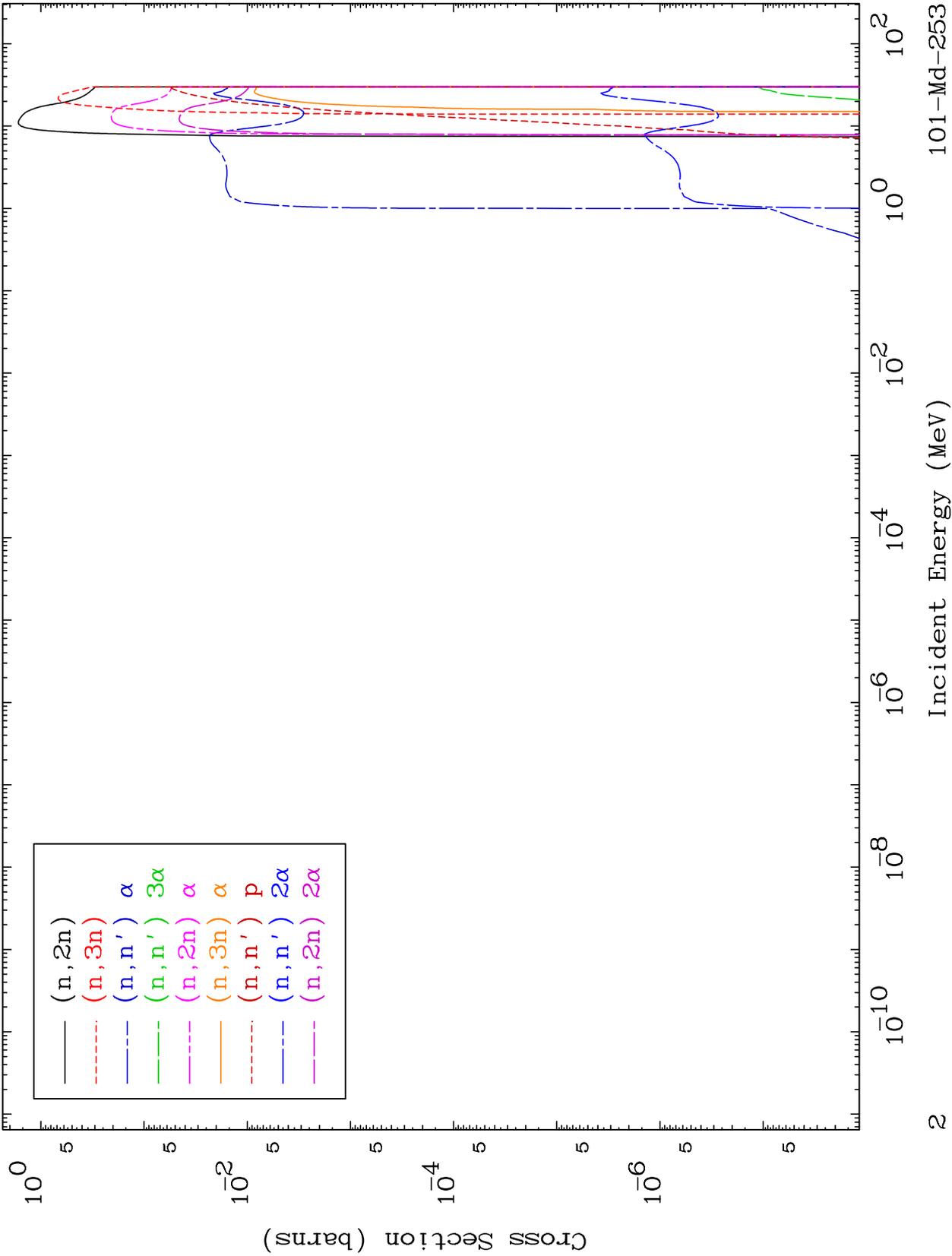
101-Md-253



MAT 9954

Neutron Production  
293 Kelvin Cross Sections

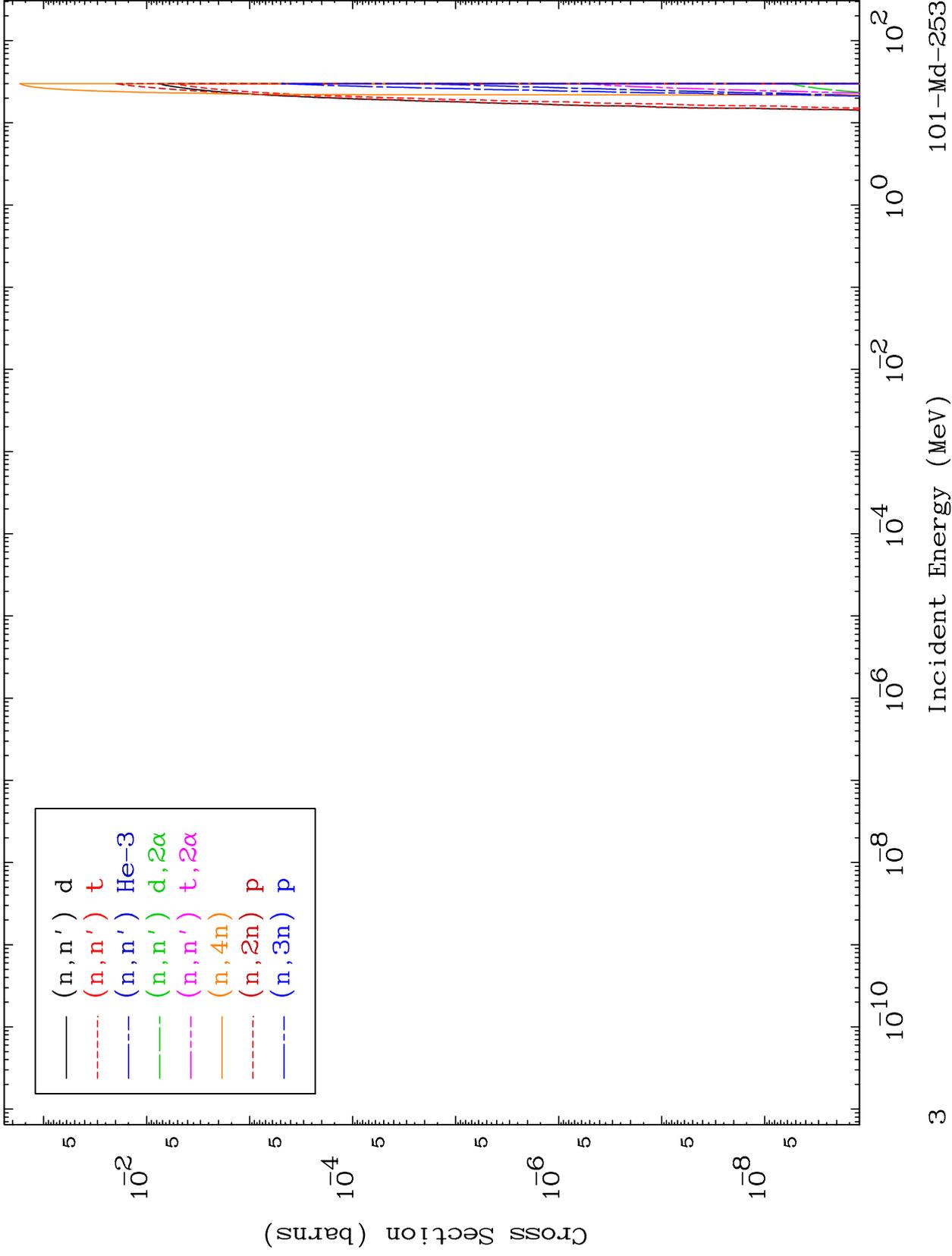
101-Md-253



MAT 9954

Neutron Production  
293 Kelvin Cross Sections

101-Md-253

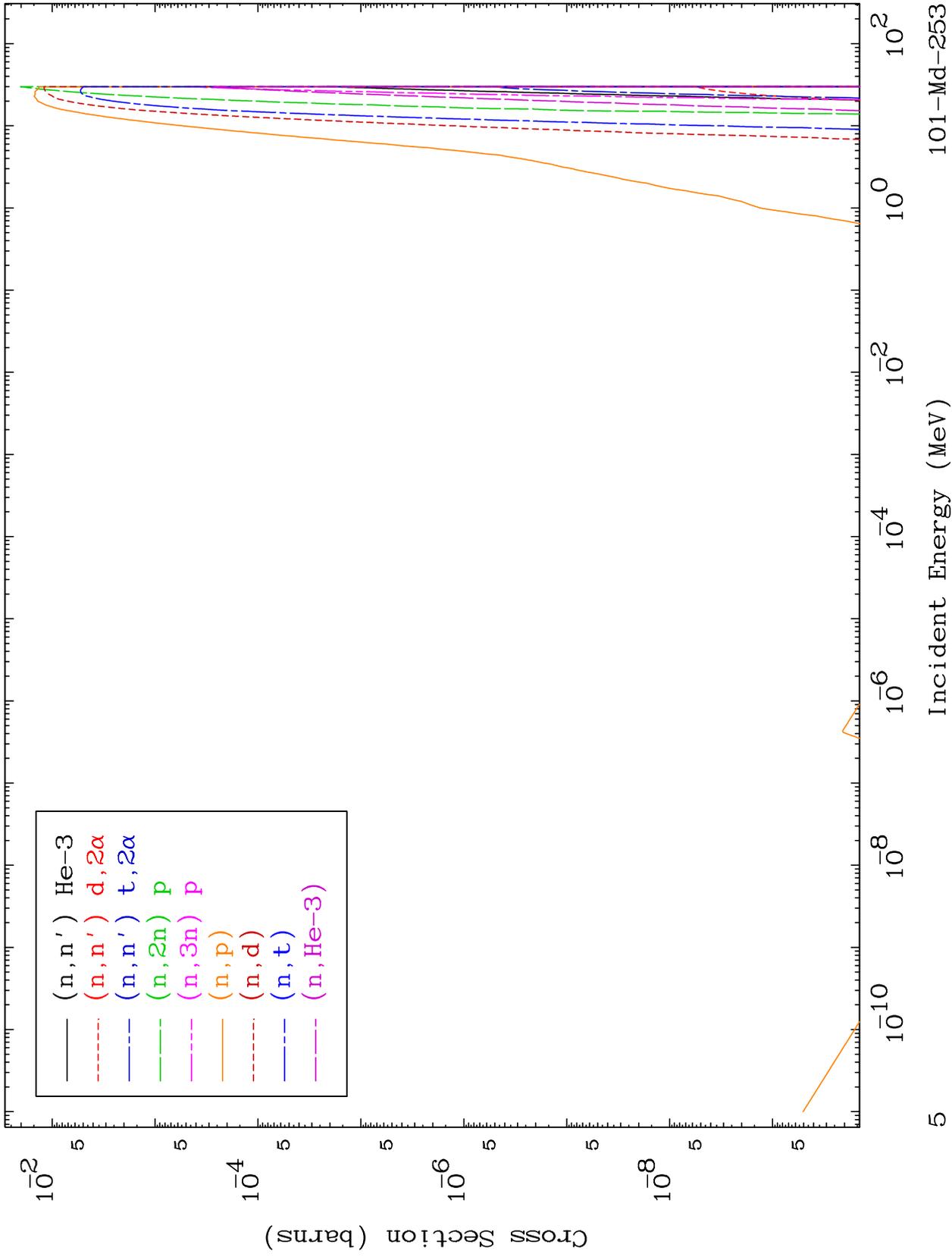




MAT 9954

Charged Particle  
293 Kelvin Cross Sections

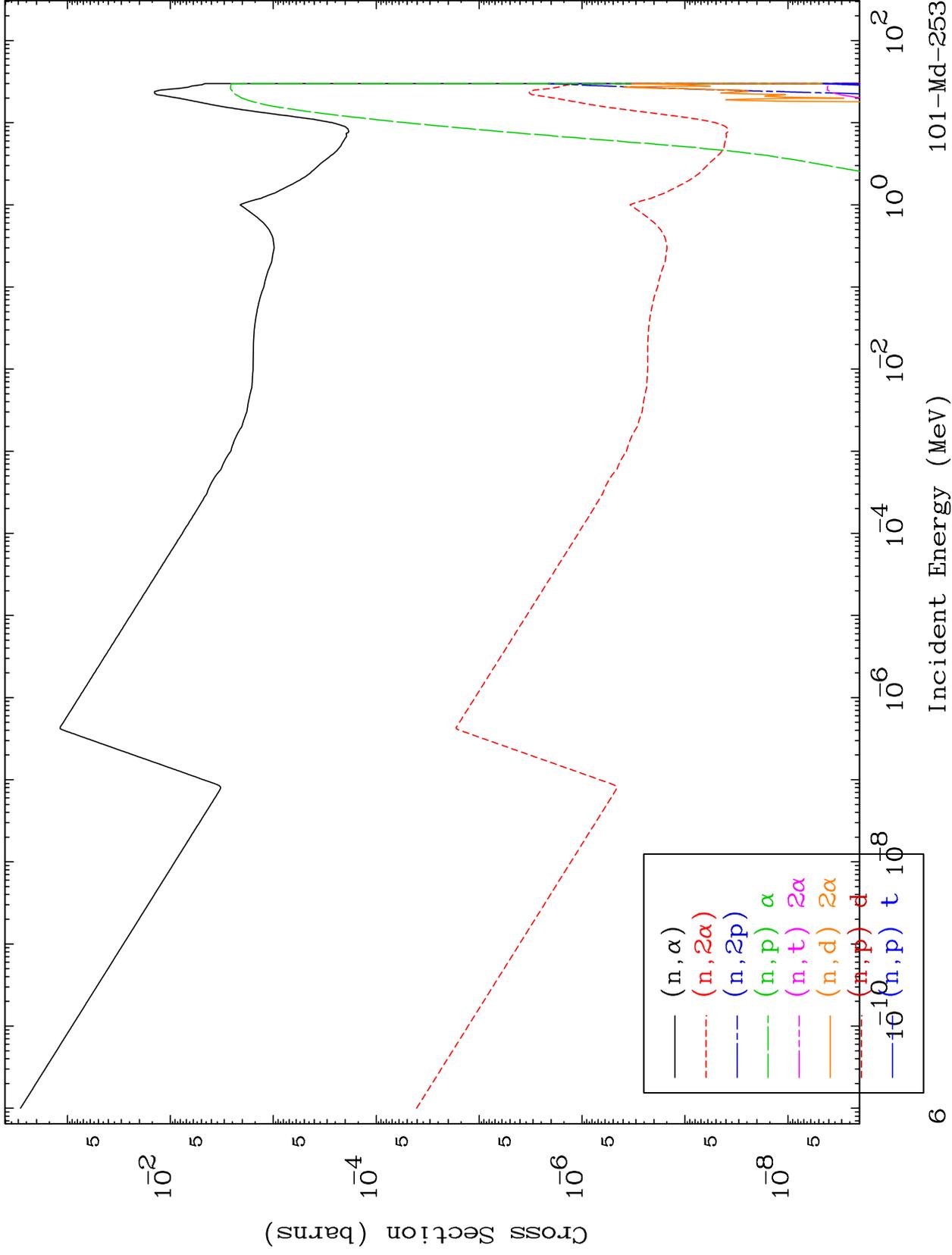
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MAT 9954

Charged Particle  
293 Kelvin Cross Sections

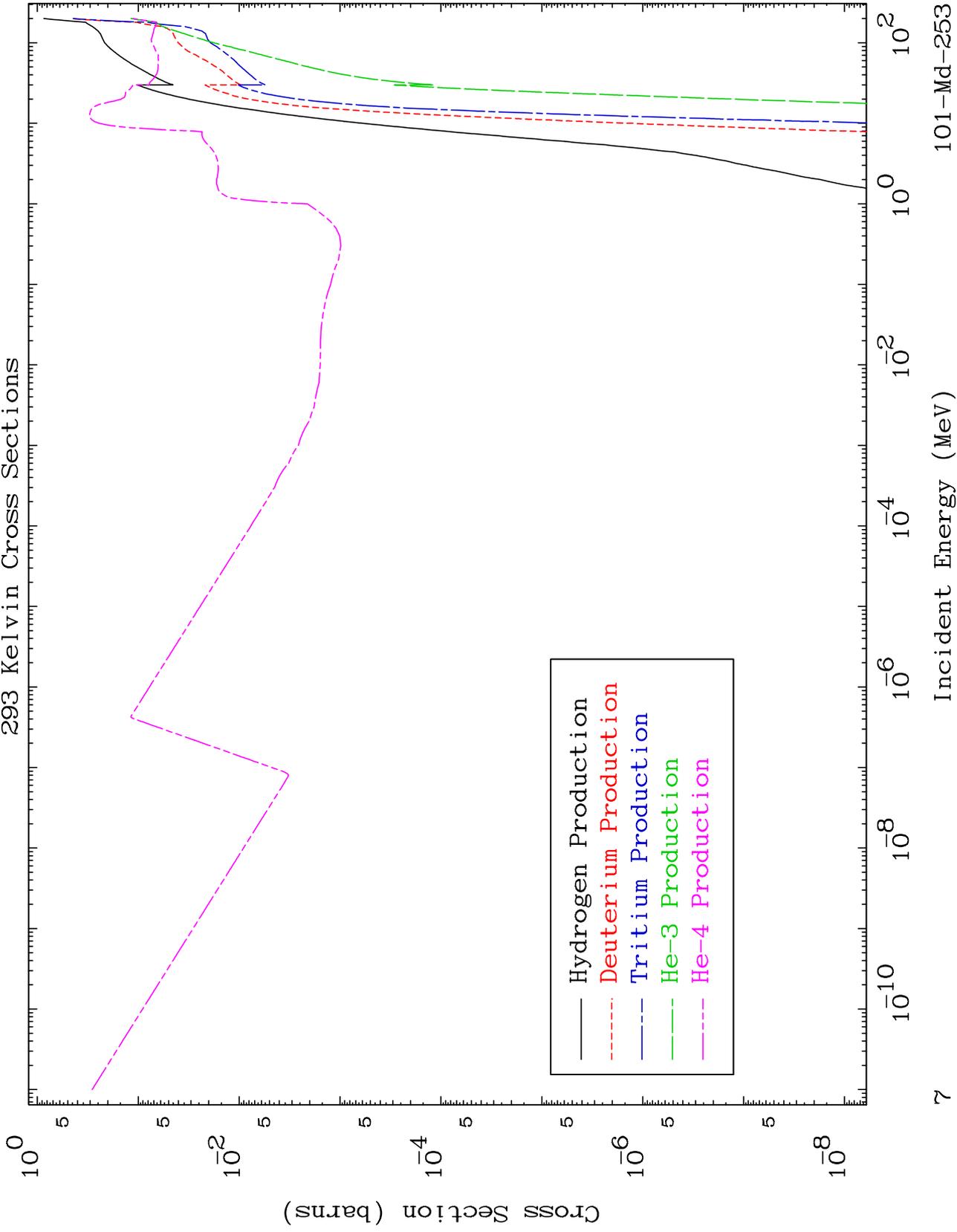
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Particle Production  
293 Kelvin Cross Sections

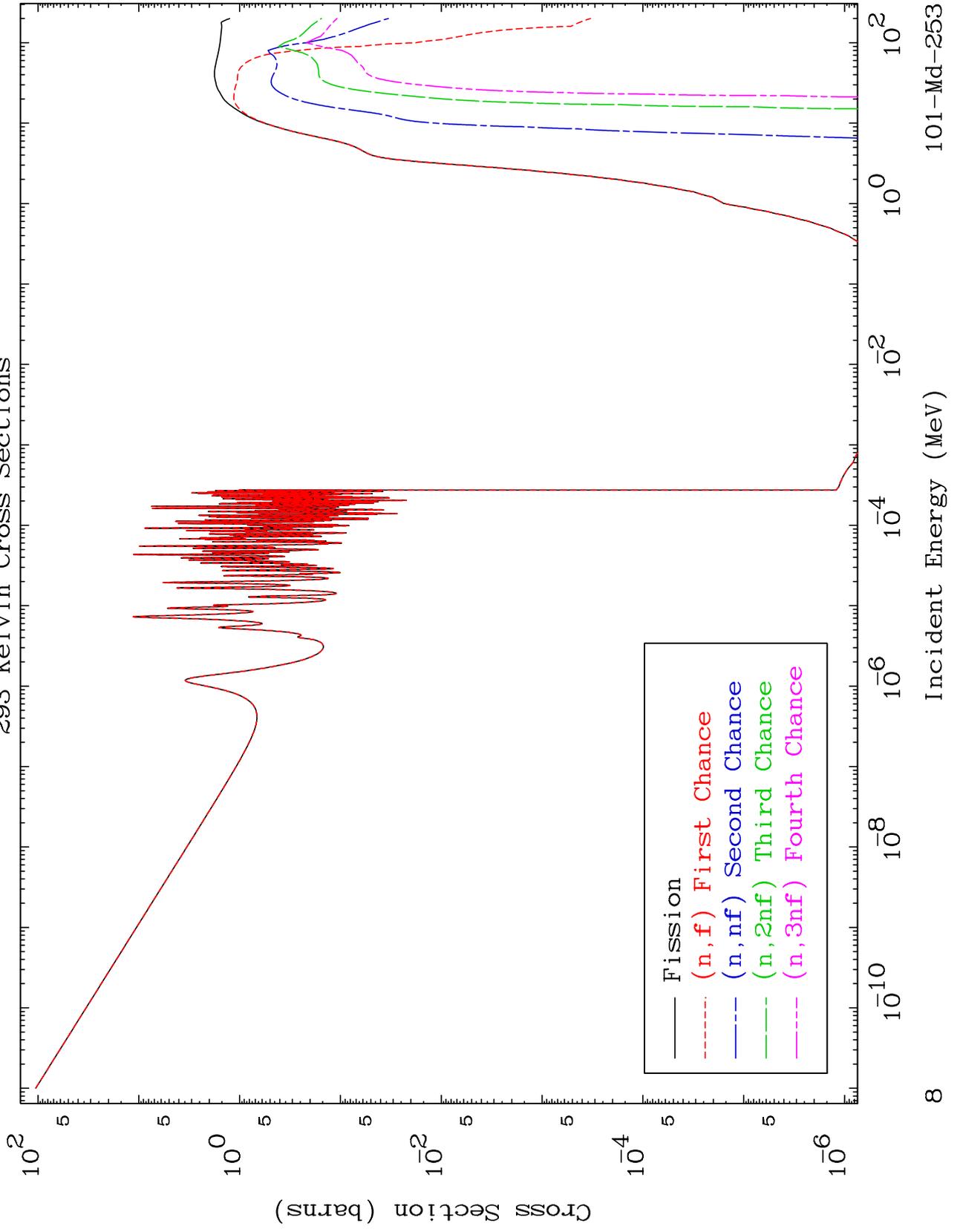
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Fission  
293 Kelvin Cross Sections

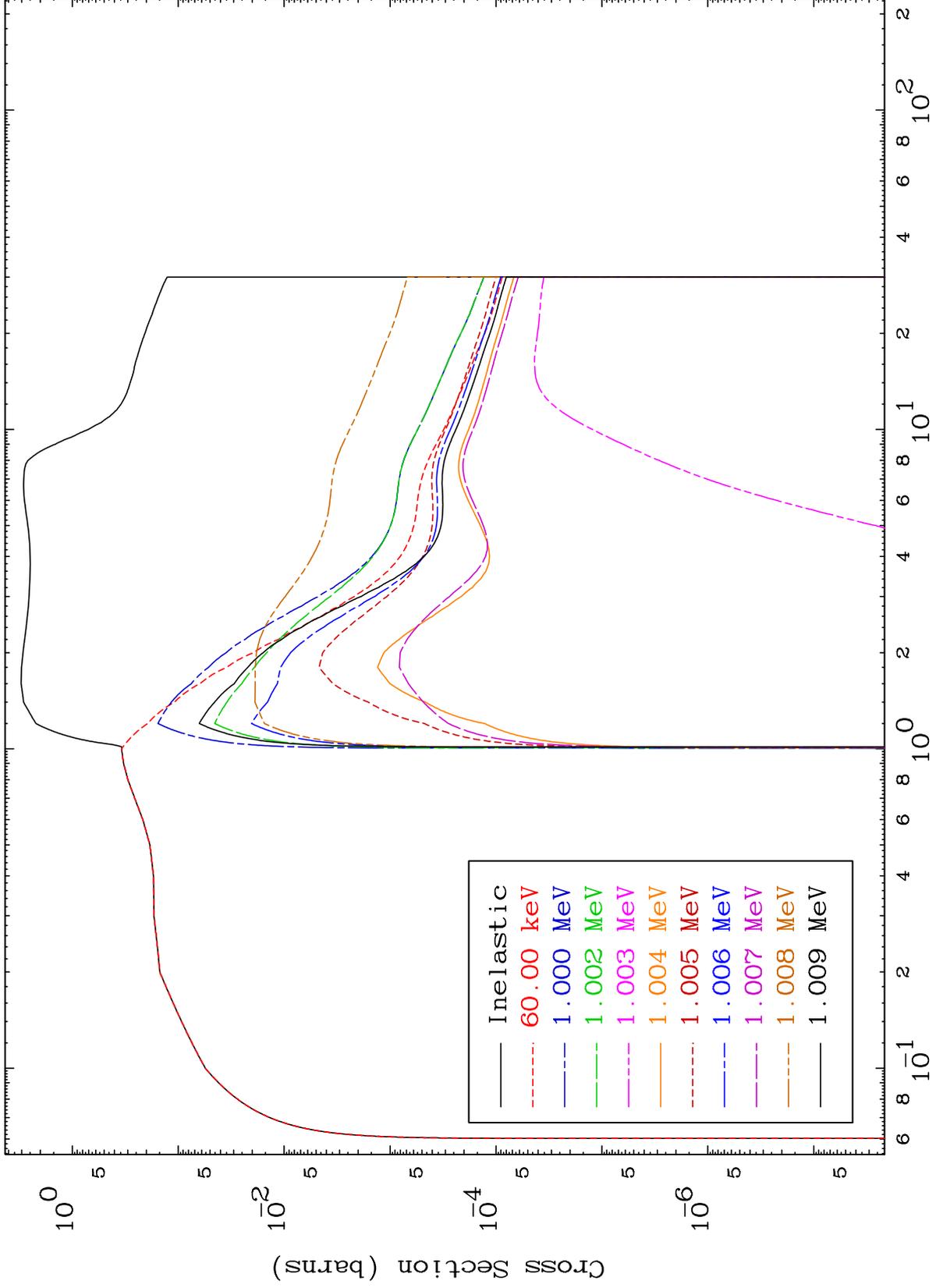
101-Md-253



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(n,n') Level  
293 Kelvin Cross Sections

101-Md-253



9

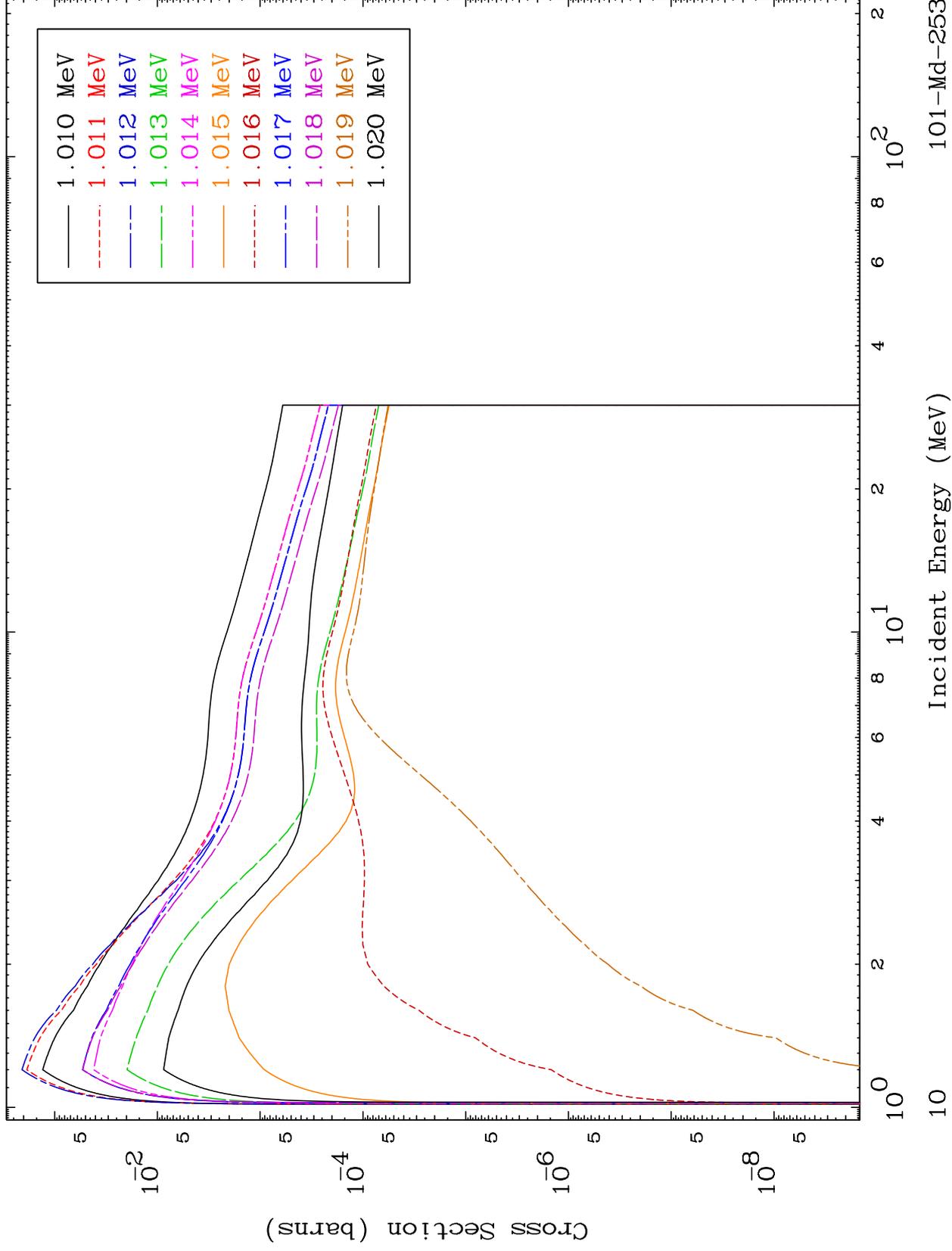
Incident Energy (MeV)

101-Md-253

MAT 9954

(n,n') Level  
293 Kelvin Cross Sections

101-Md-253

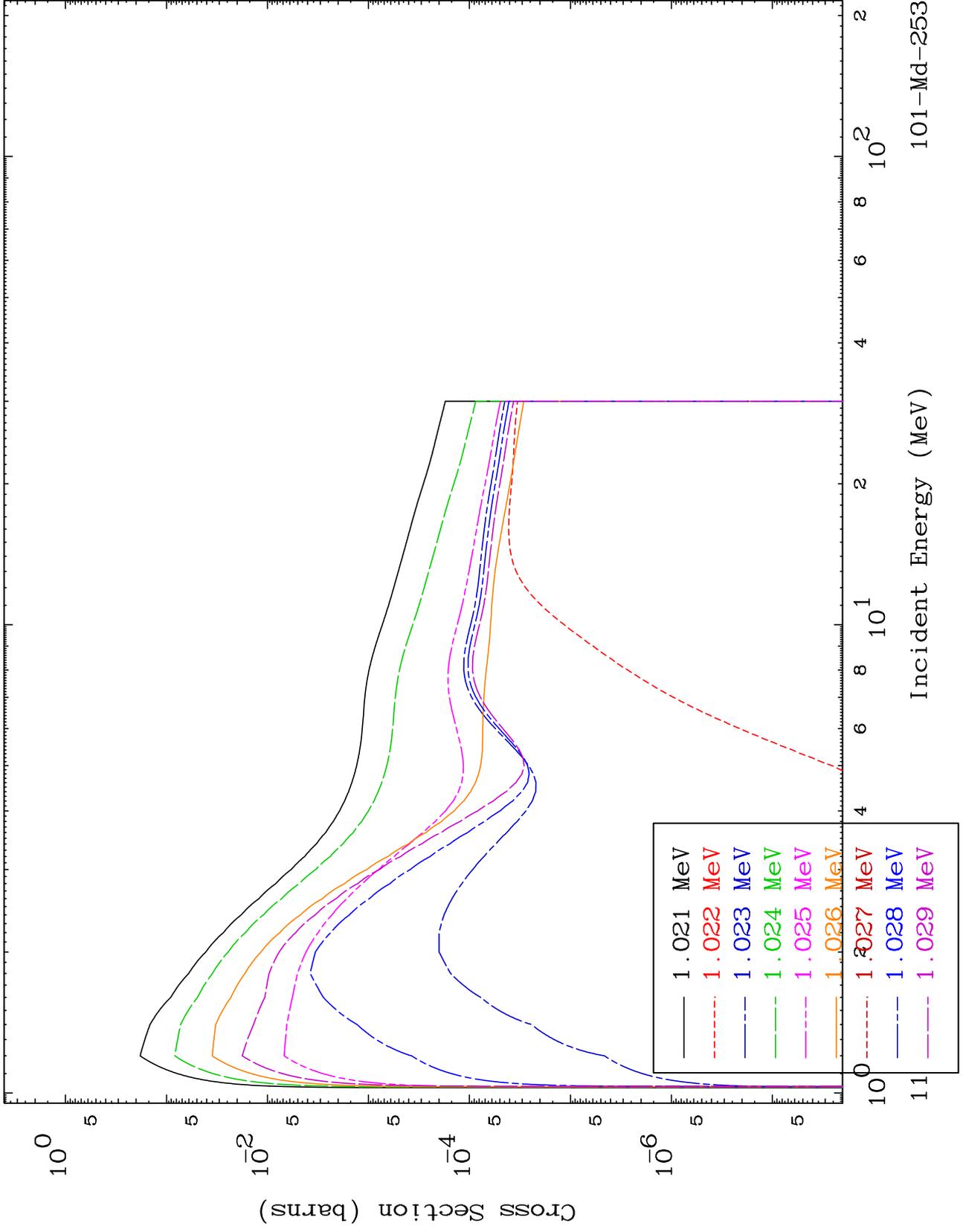


101-Md-253

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(n,n') Level  
293 Kelvin Cross Sections

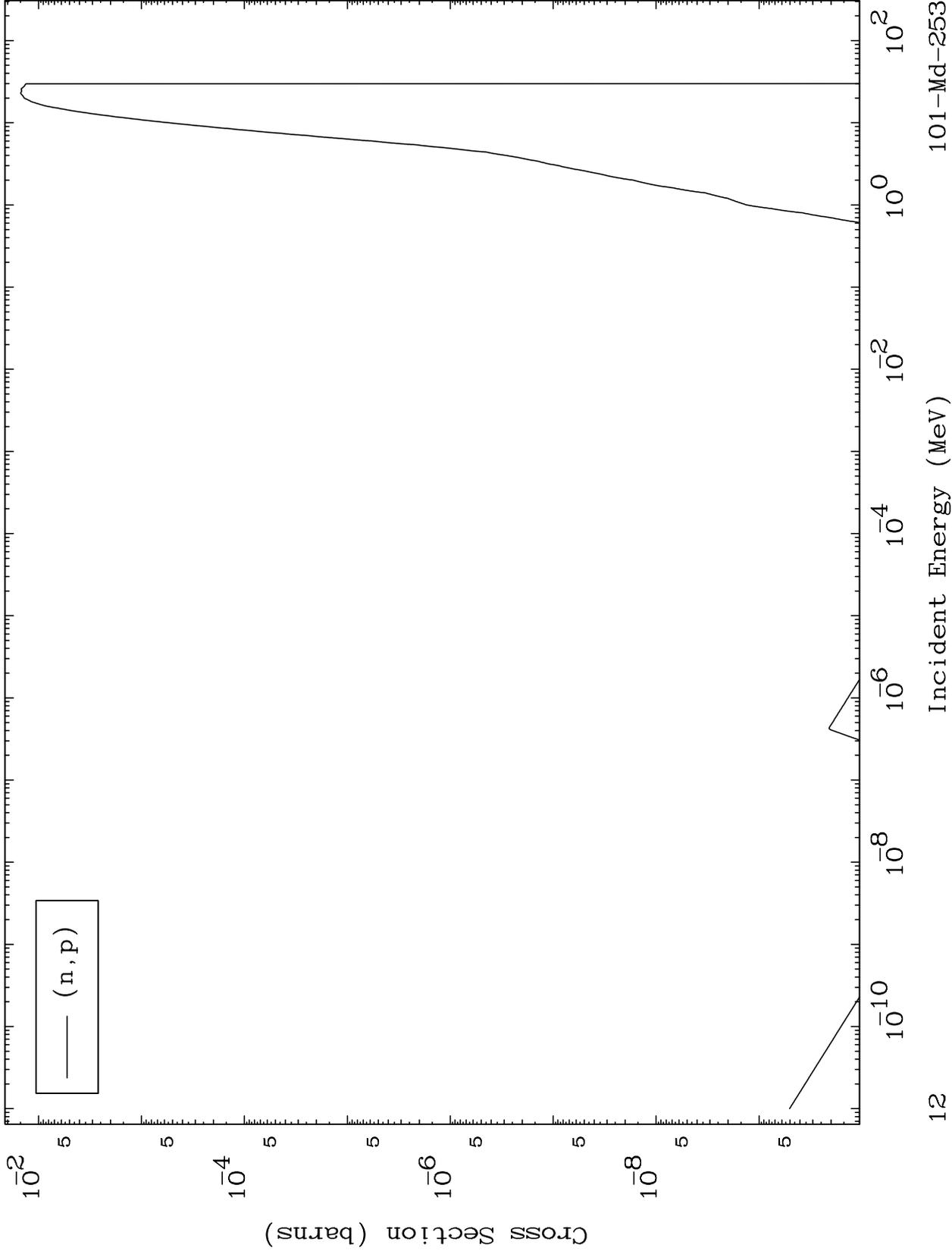
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(n,p) Levels  
293 Kelvin Cross Sections

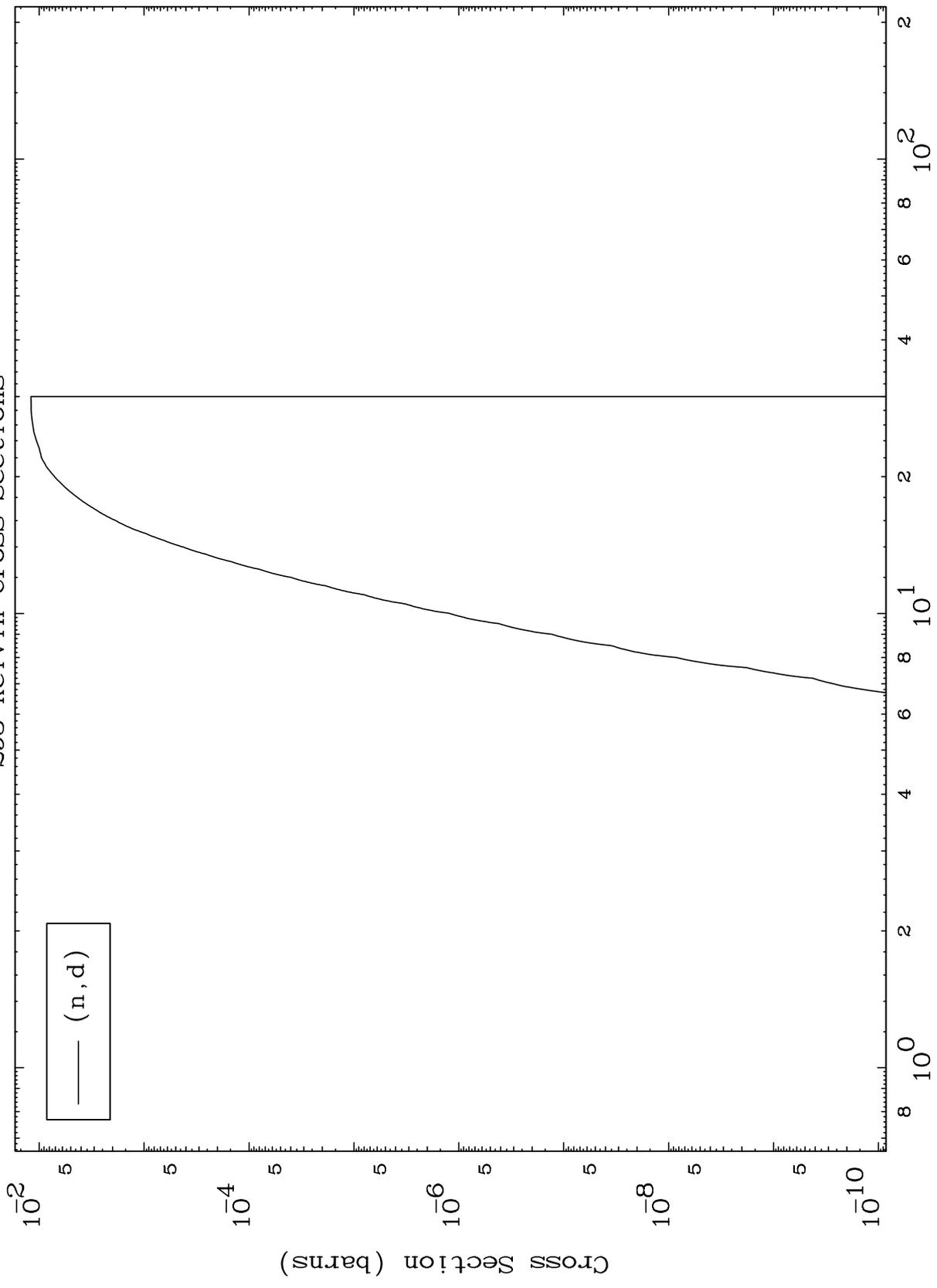
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MAT 9954

(n,d) Levels  
293 Kelvin Cross Sections

101-Md-253



13

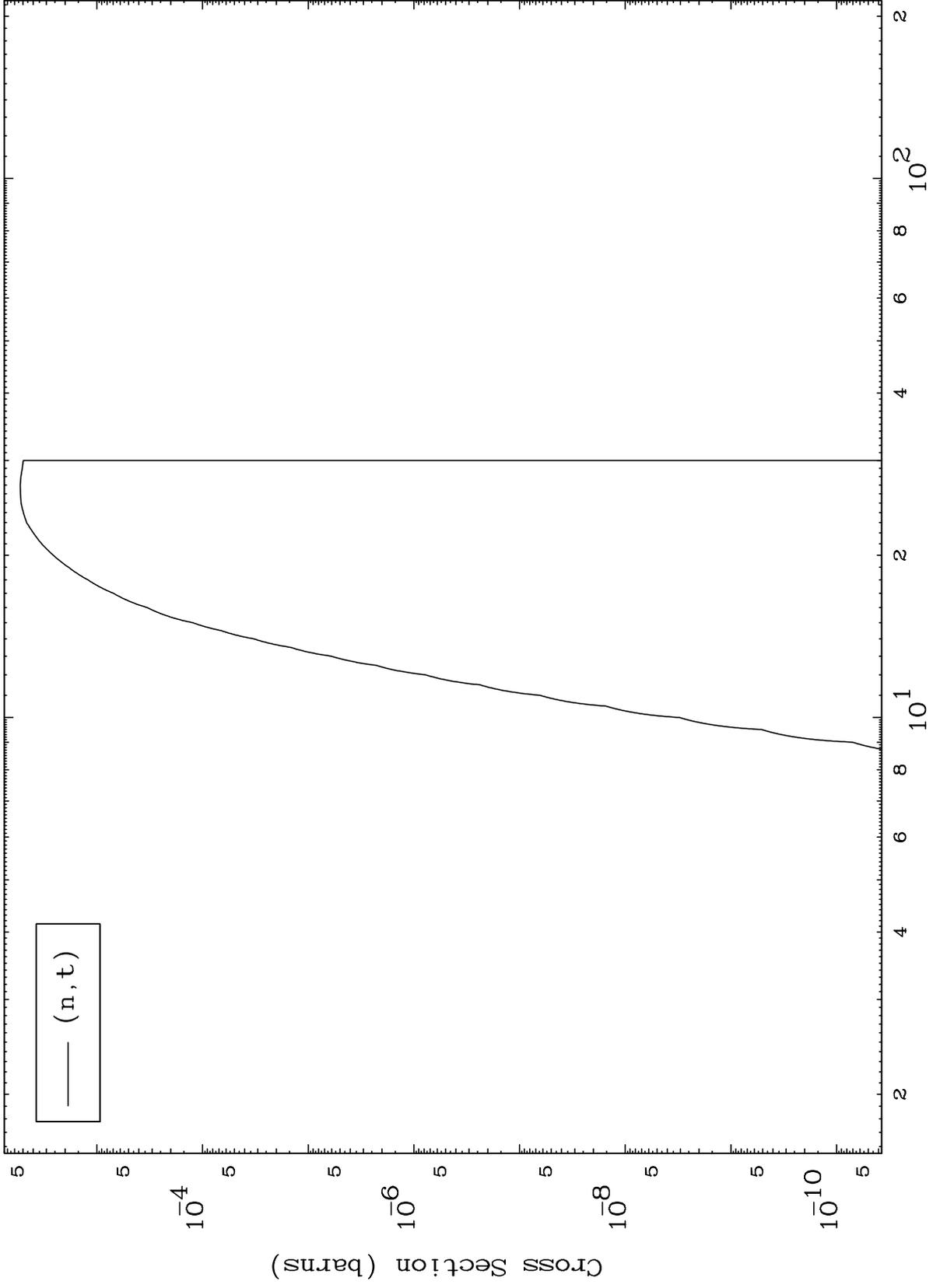
Incident Energy (MeV)

101-Md-253

MAT 9954

(n,t) Levels  
293 Kelvin Cross Sections

101-Md-253



14

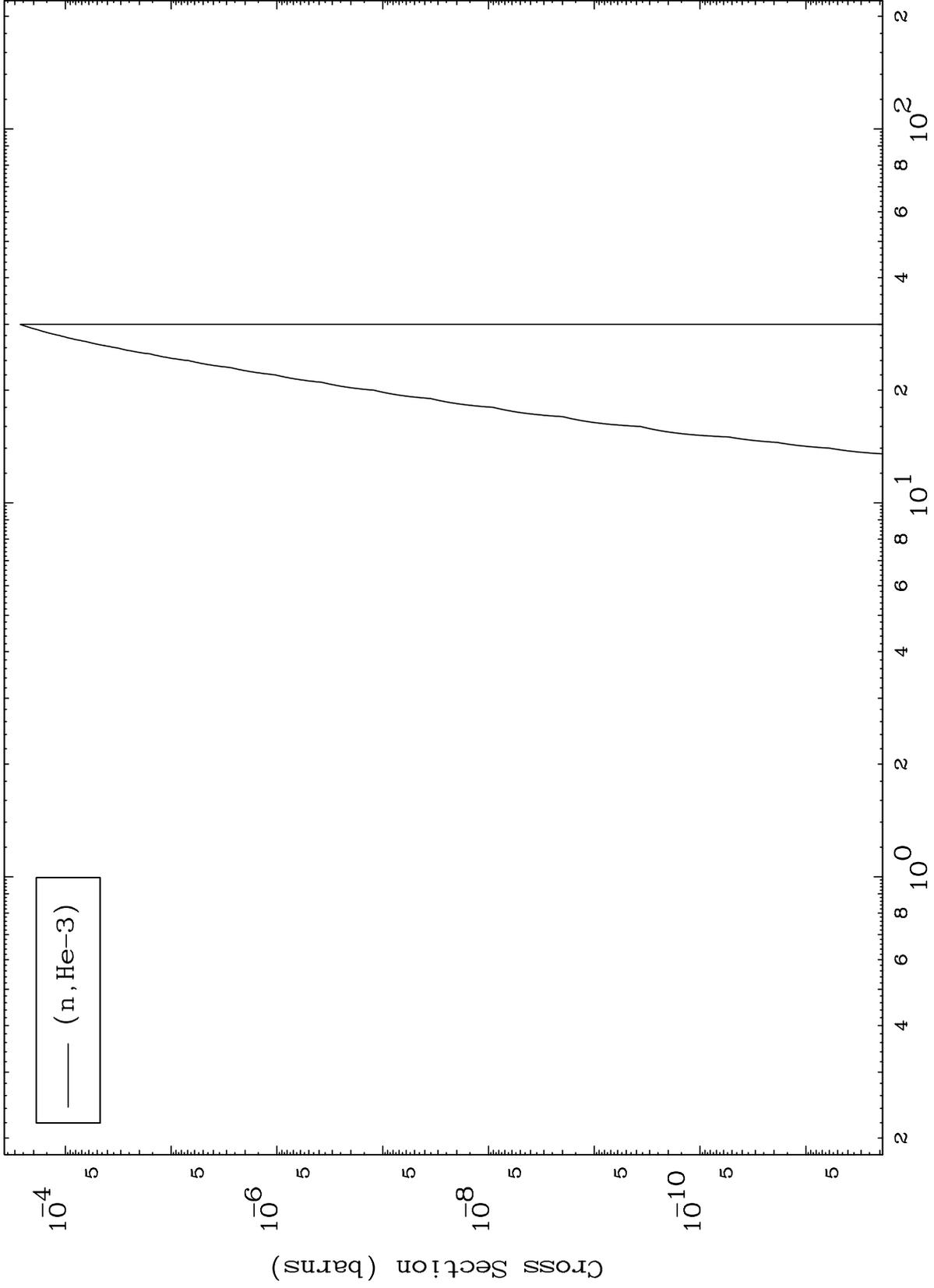
Incident Energy (MeV)

101-Md-253

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(n,He3) Levels  
293 Kelvin Cross Sections

101-Md-253



15

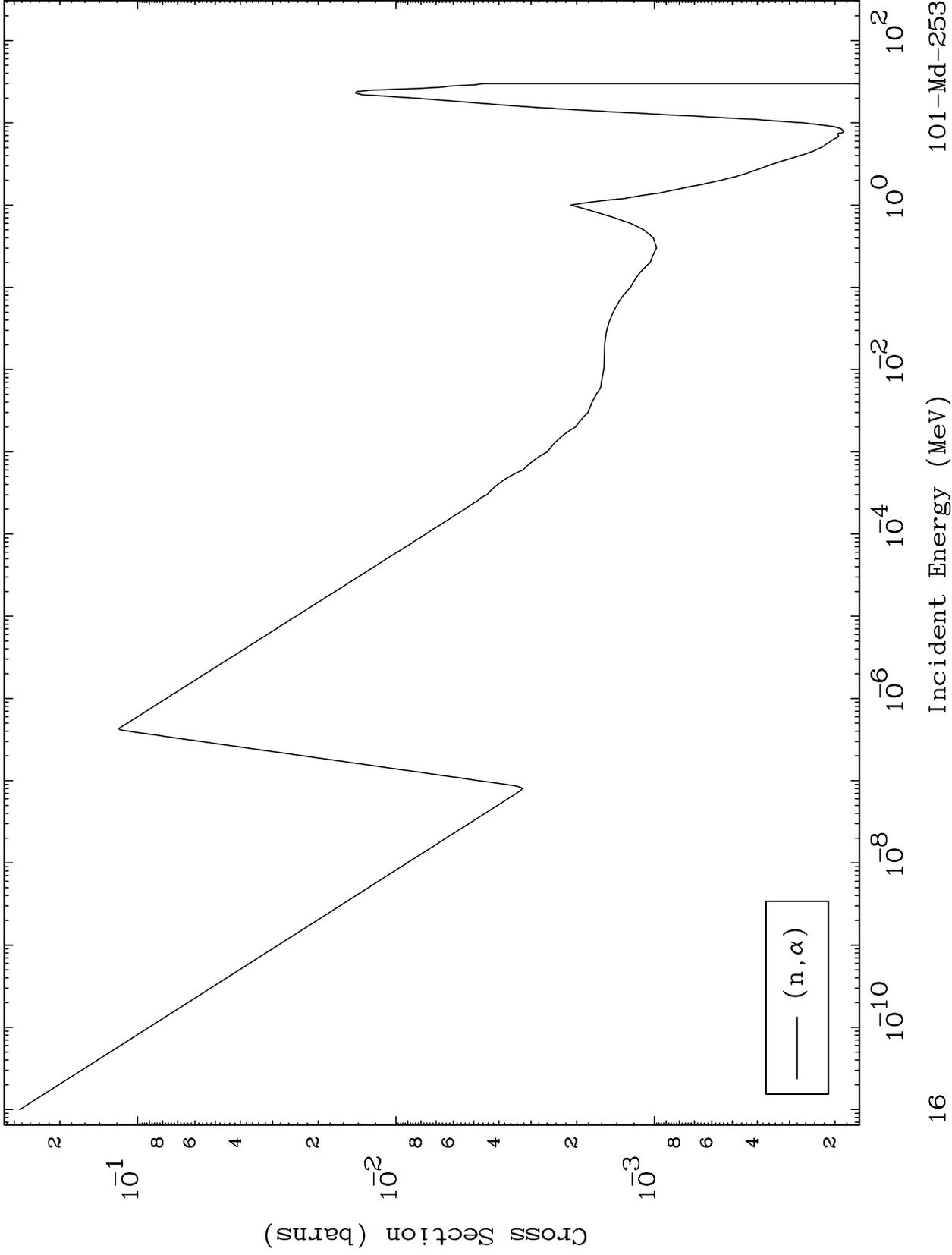
Incident Energy (MeV)

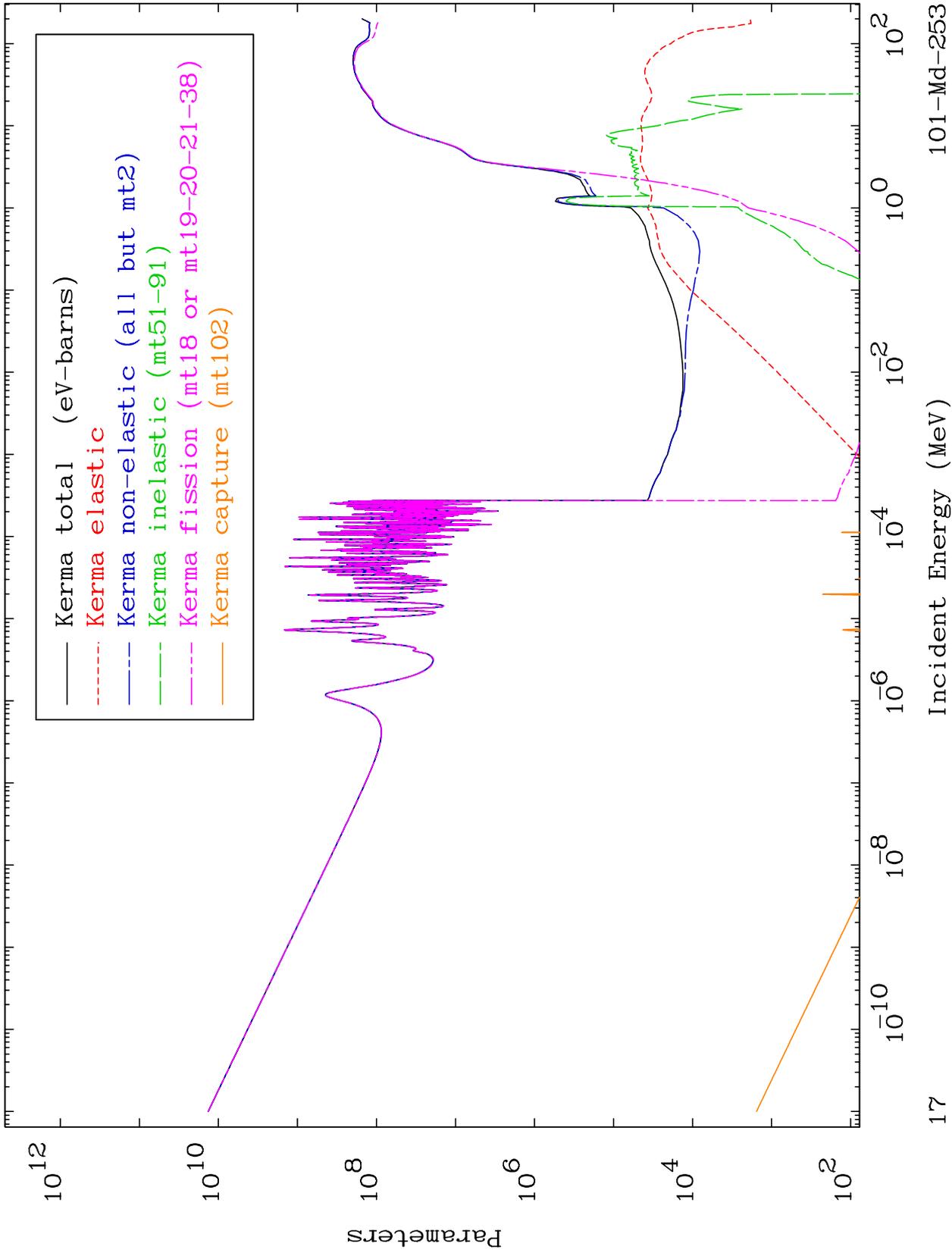
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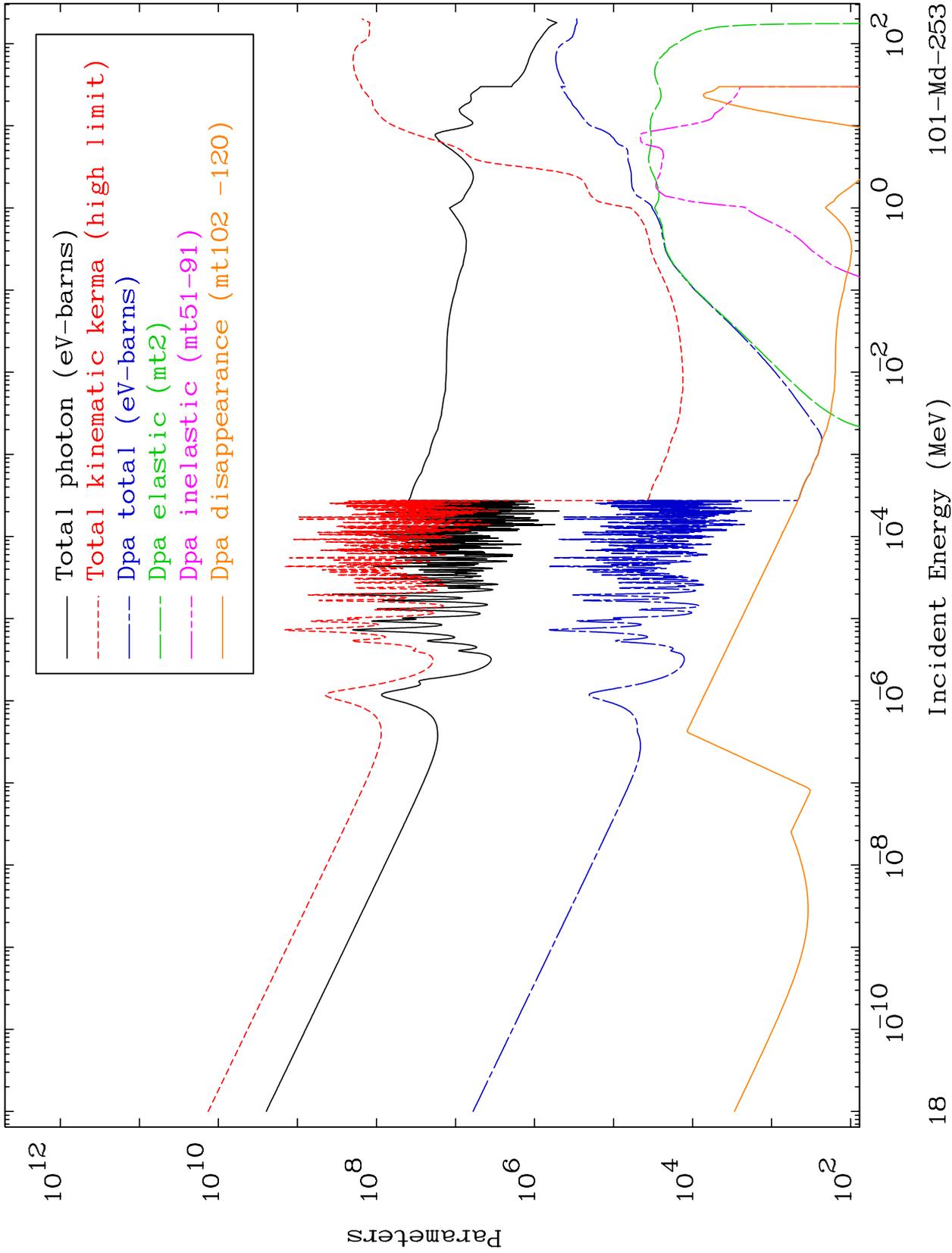
MAT 9954

(n,α) Levels  
293 Kelvin Cross Sections

101-Md-253



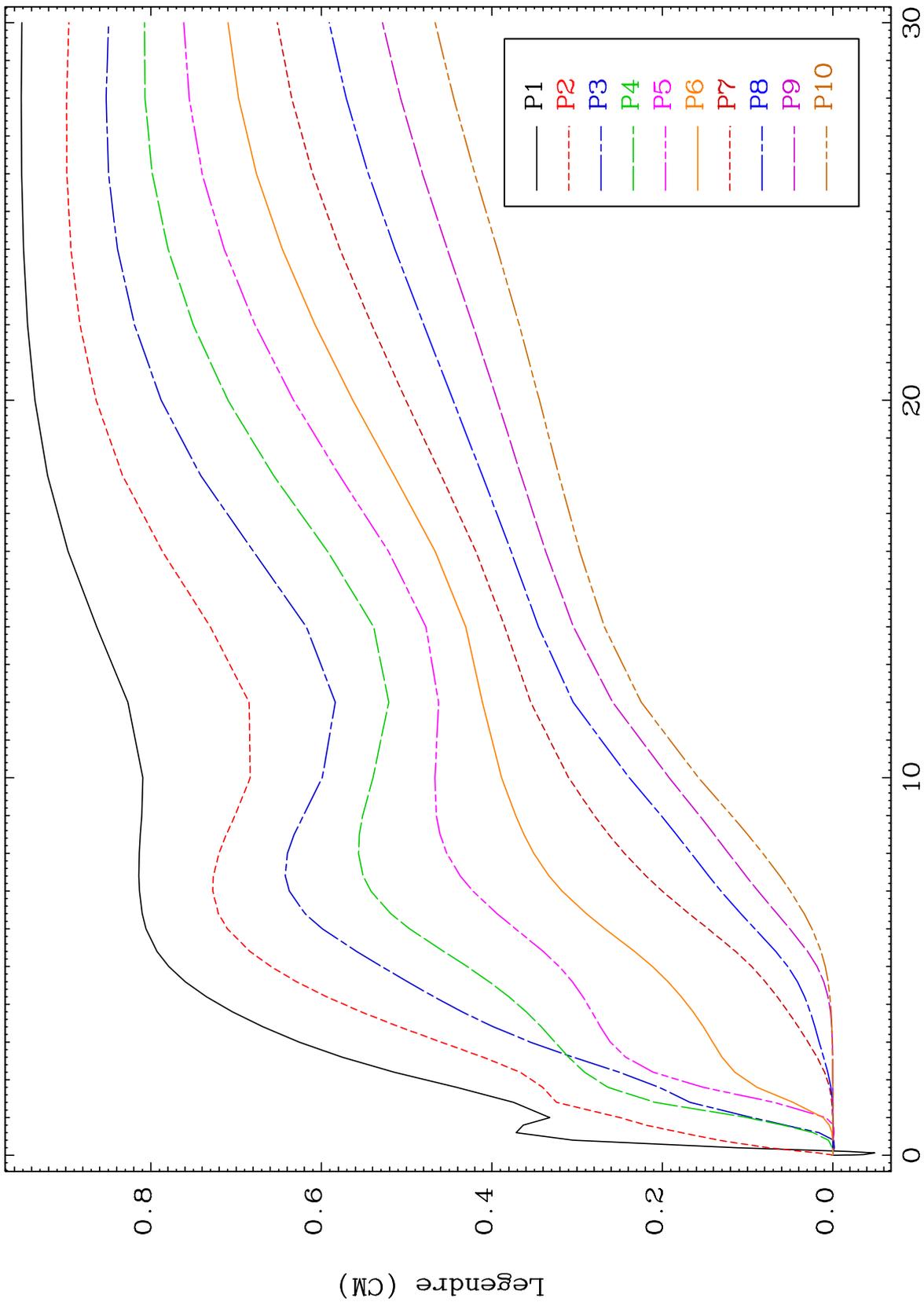




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Elastic Legendre Coefficients

101-Md-253



19

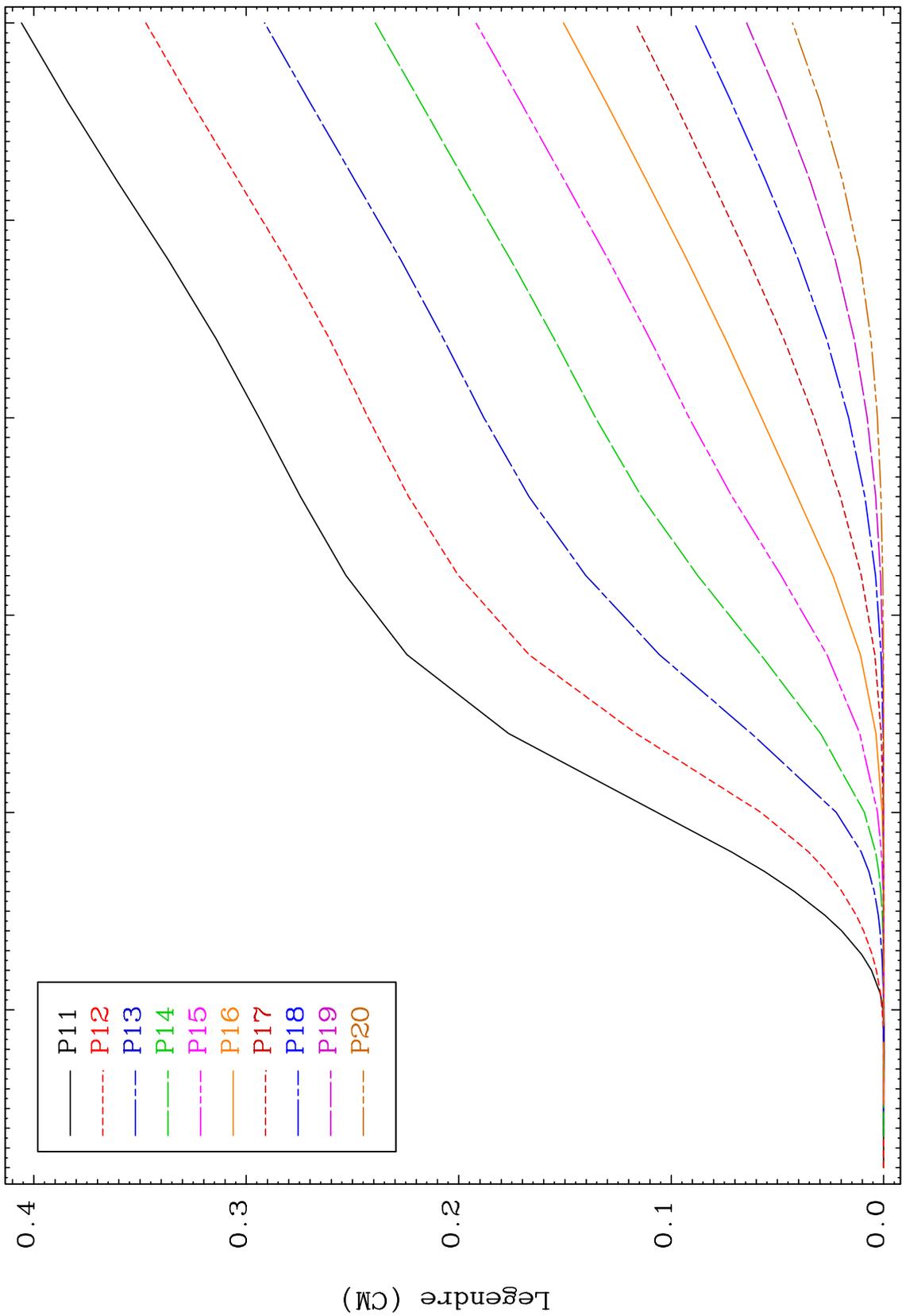
Incident Energy (MeV)

101-Md-253

MAT 9954

Elastic Legendre Coefficients

101-Md-253



20

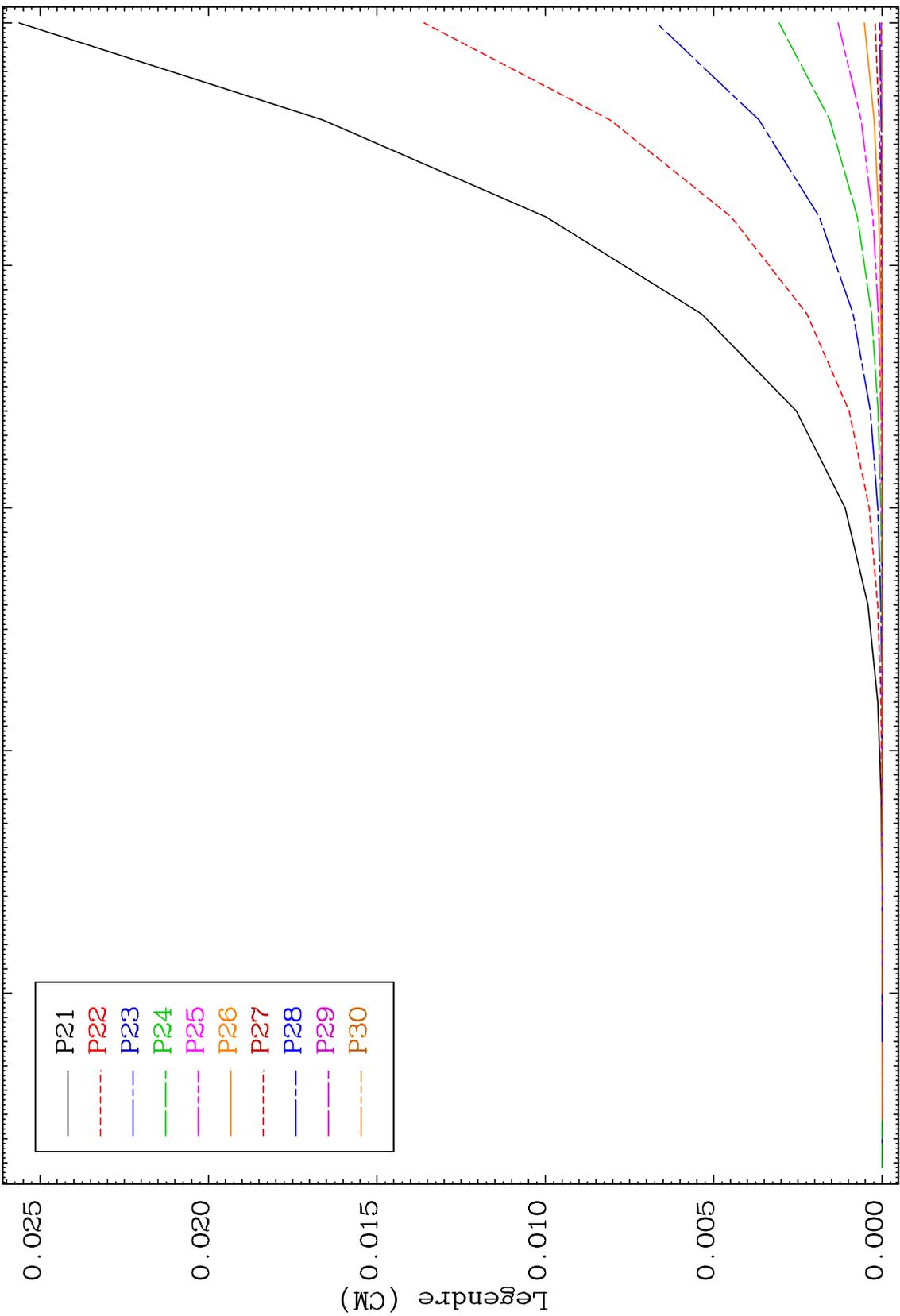
Incident Energy (MeV)

101-Md-253

MAT 9954

### Elastic Legendre Coefficients

101-Md-253



21

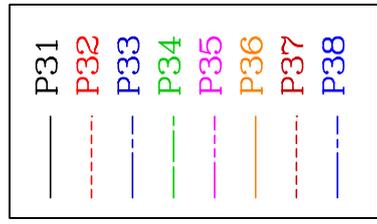
Incident Energy (MeV)

101-Md-253

MAT 9954

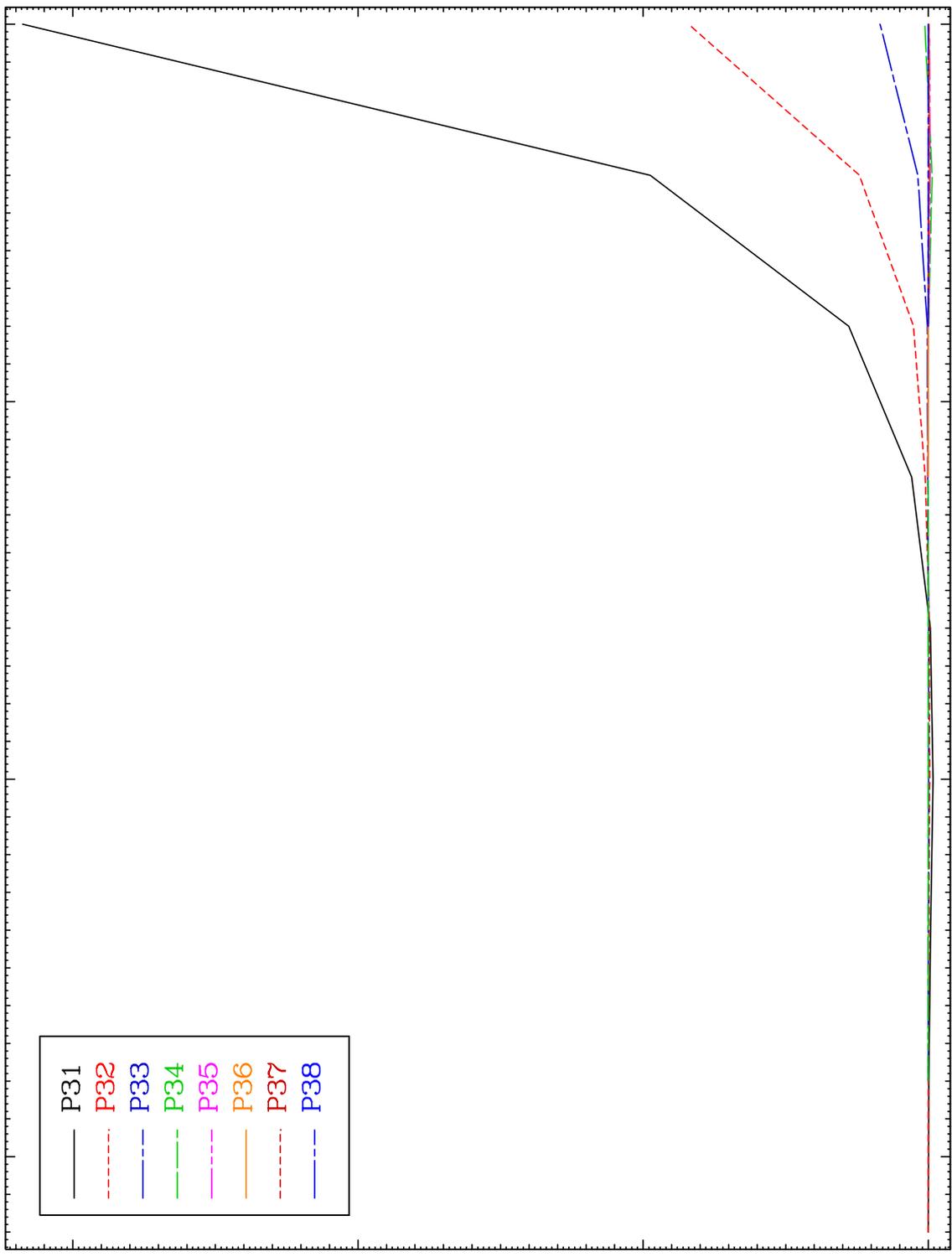
Elastic Legendre Coefficients

101-Md-253



$\times 10^{-6}$

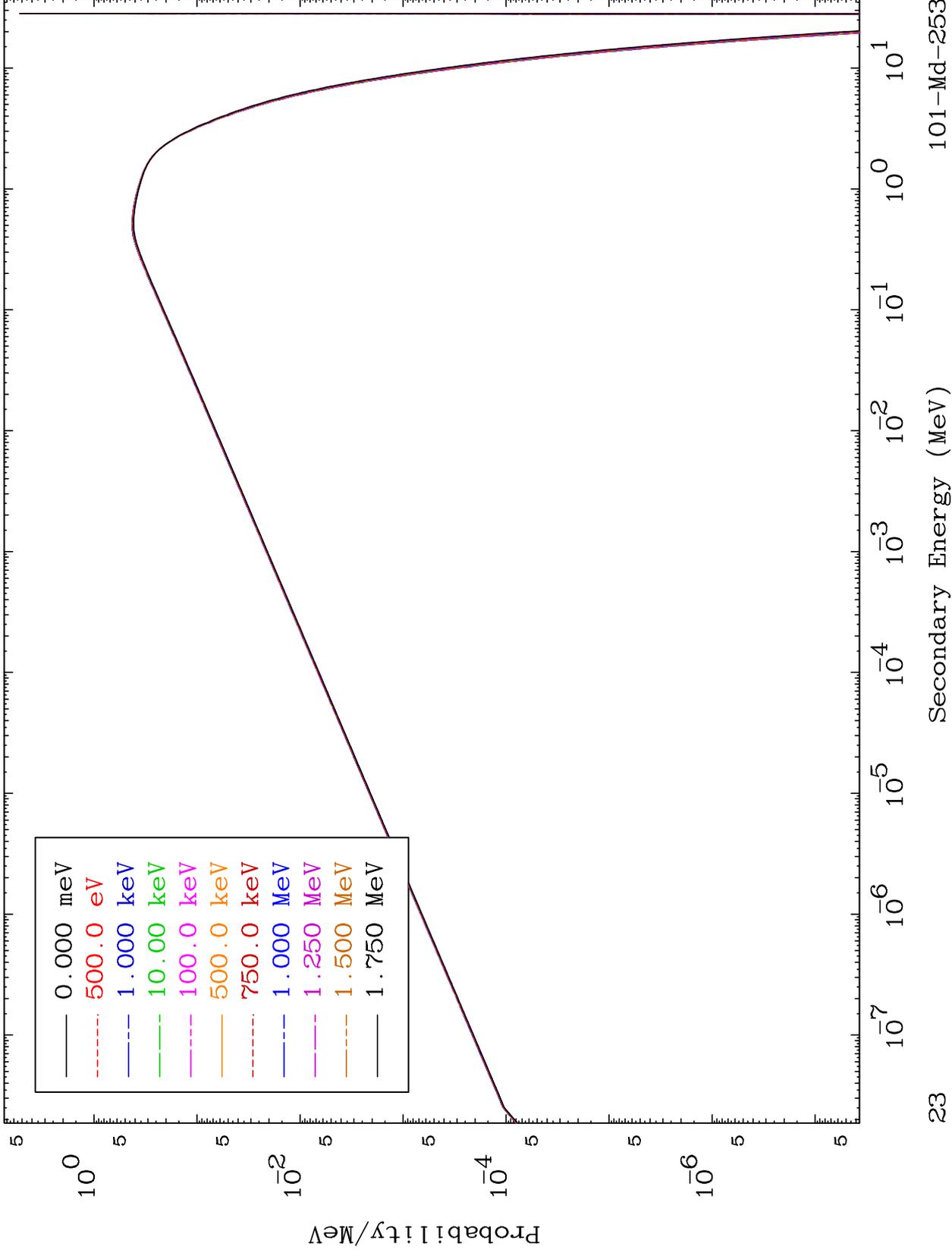
Legendre (CM)



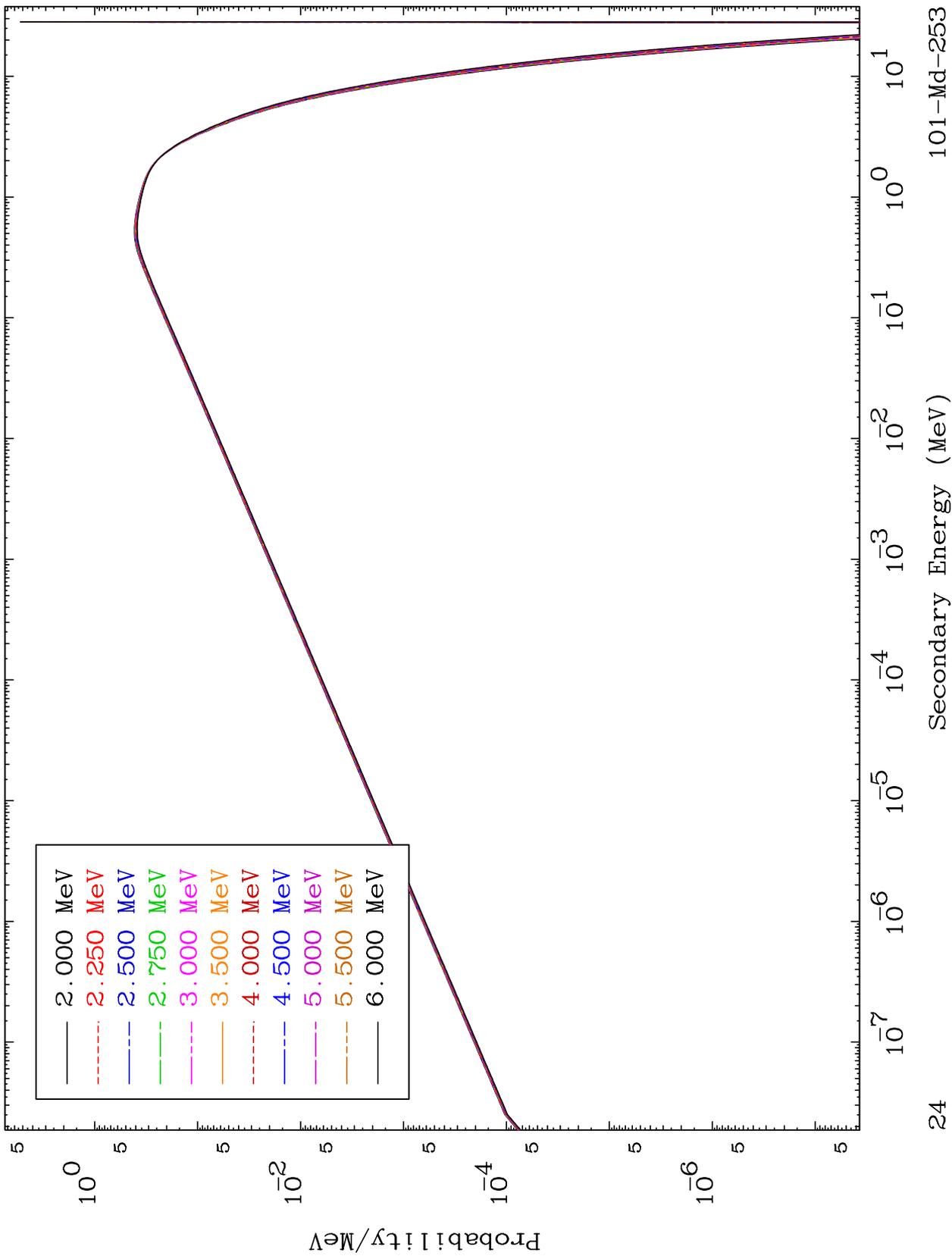
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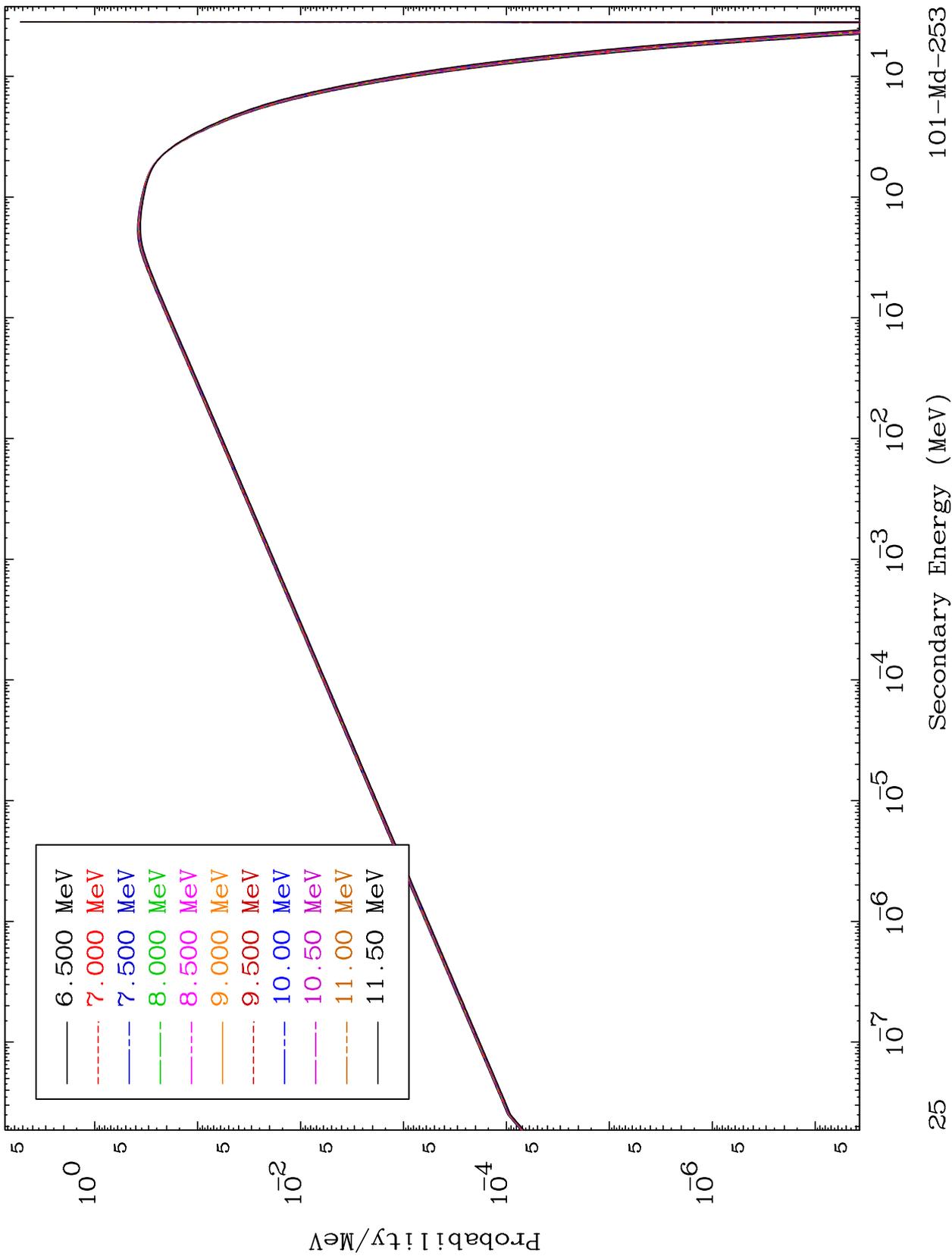
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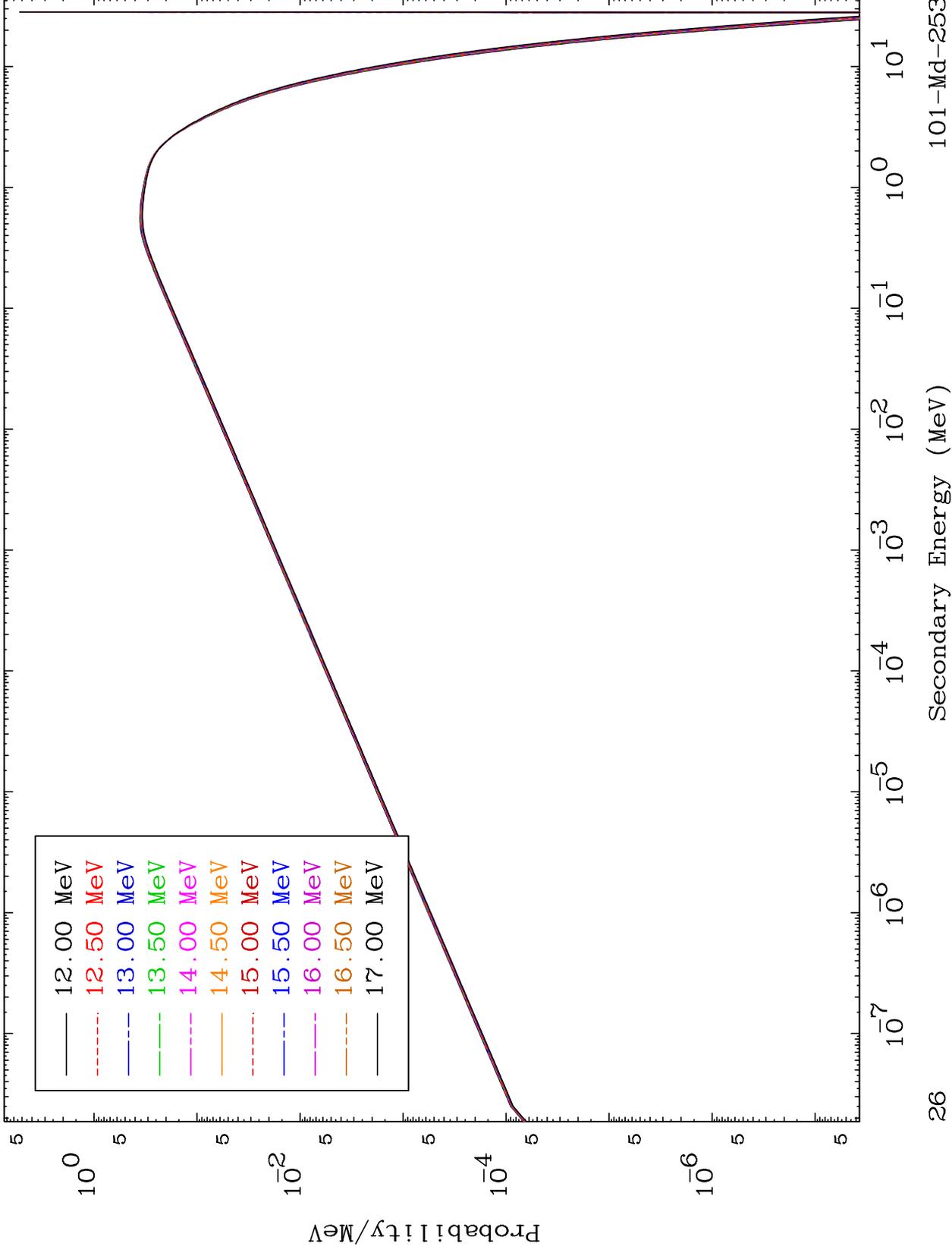
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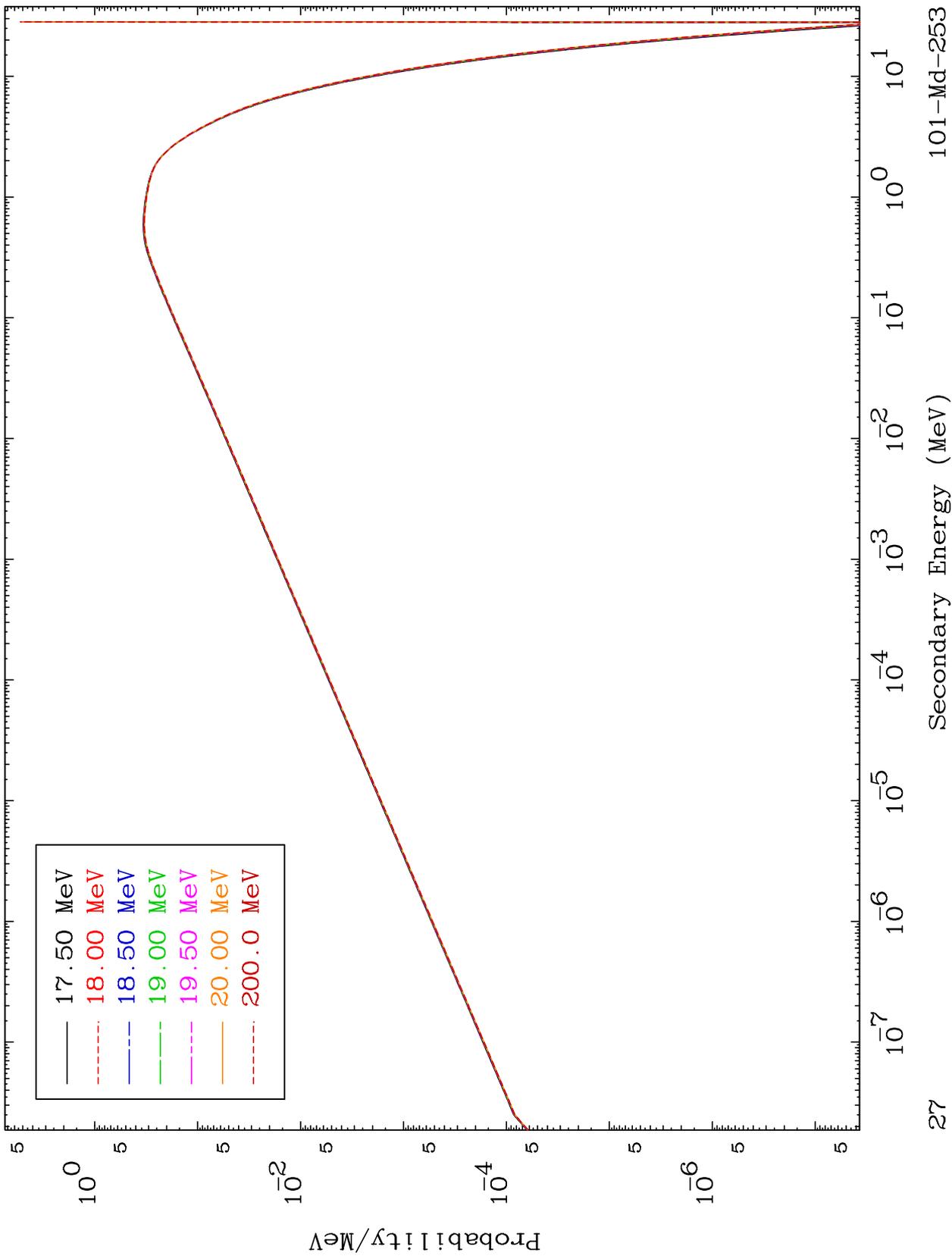


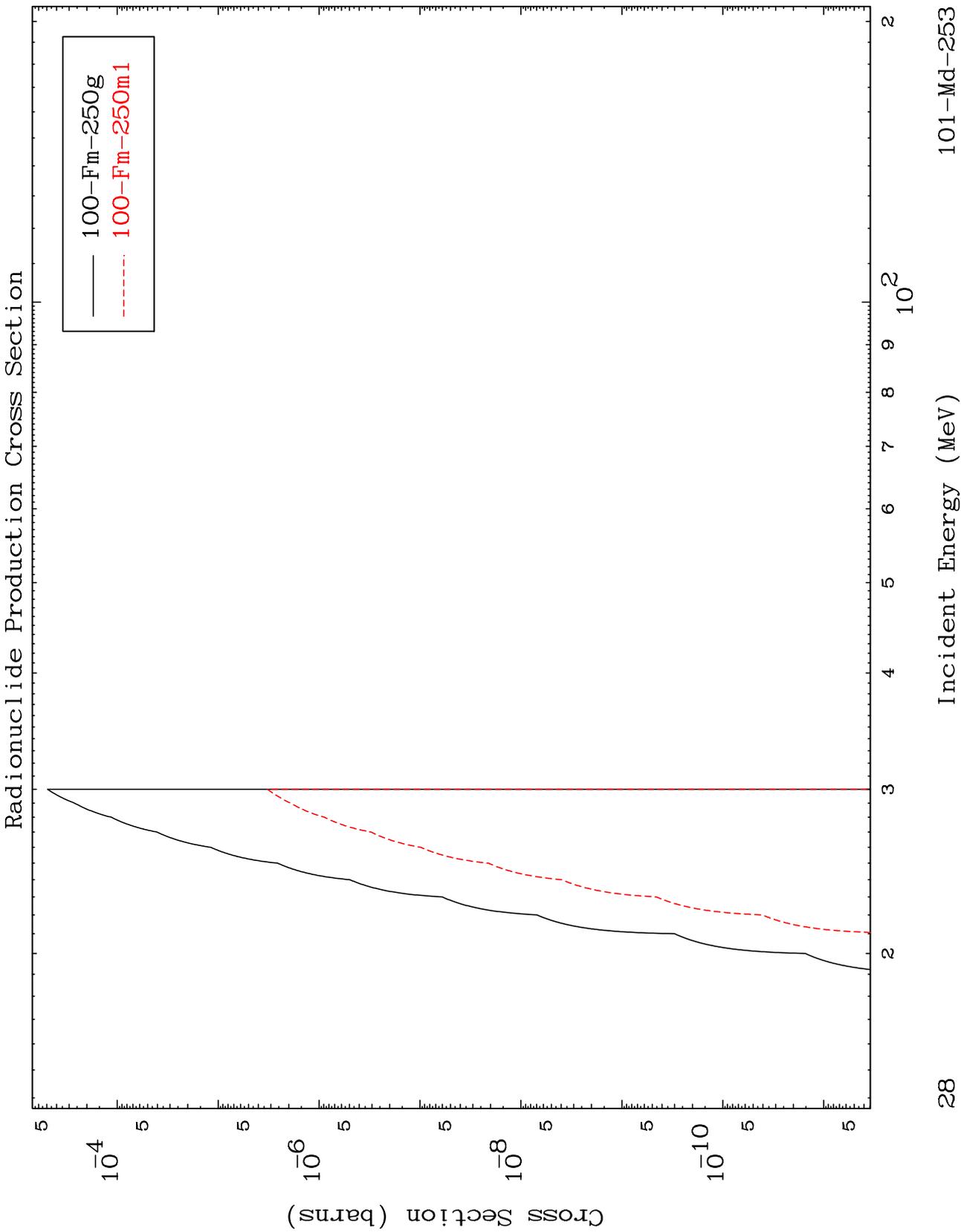
Fission  
Energy Distribution









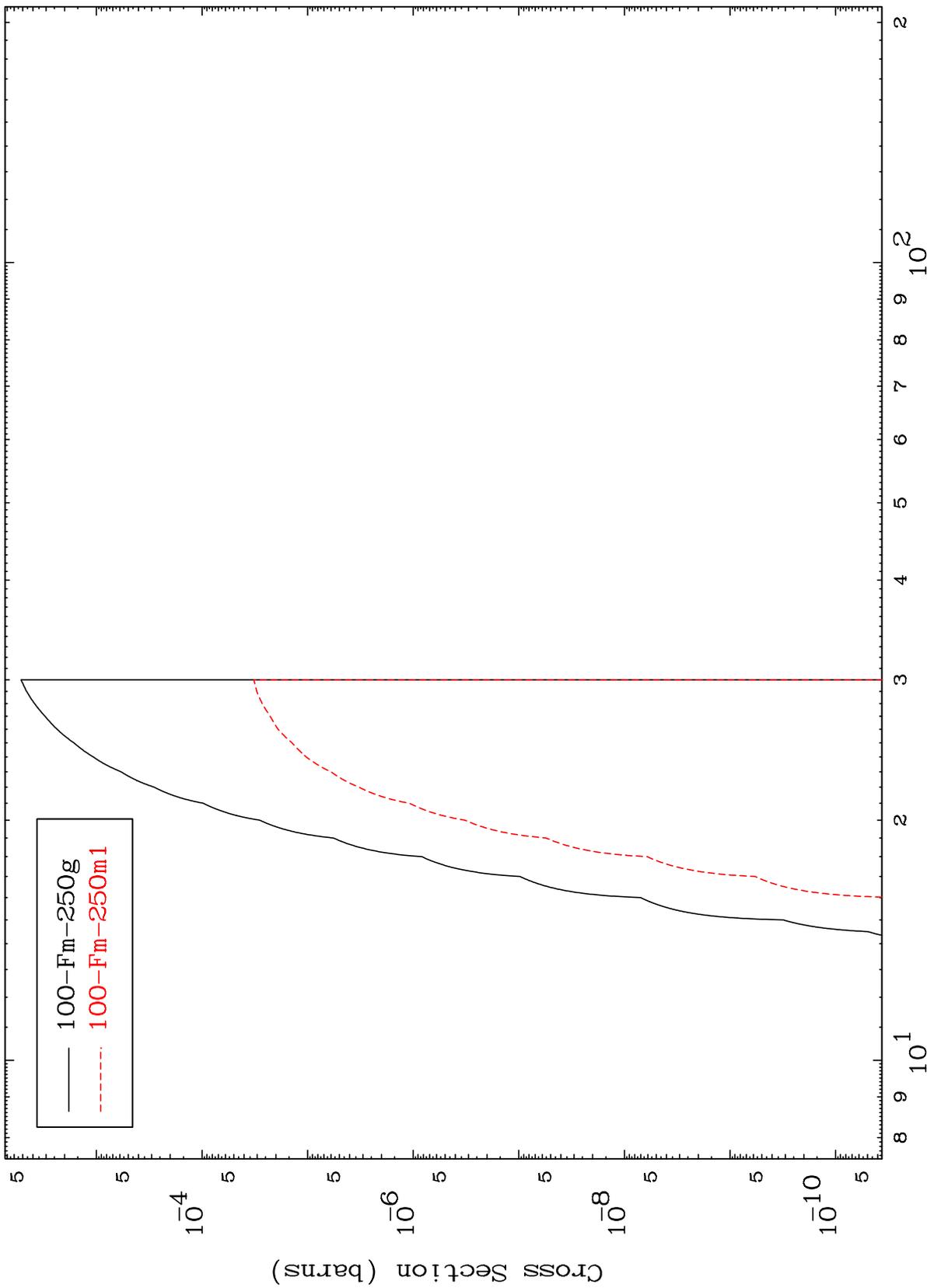


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(n,n') t

101-Md-253

Radionuclide Production Cross Section



100-Fm-250g  
100-Fm-250m1

29

Incident Energy (MeV)

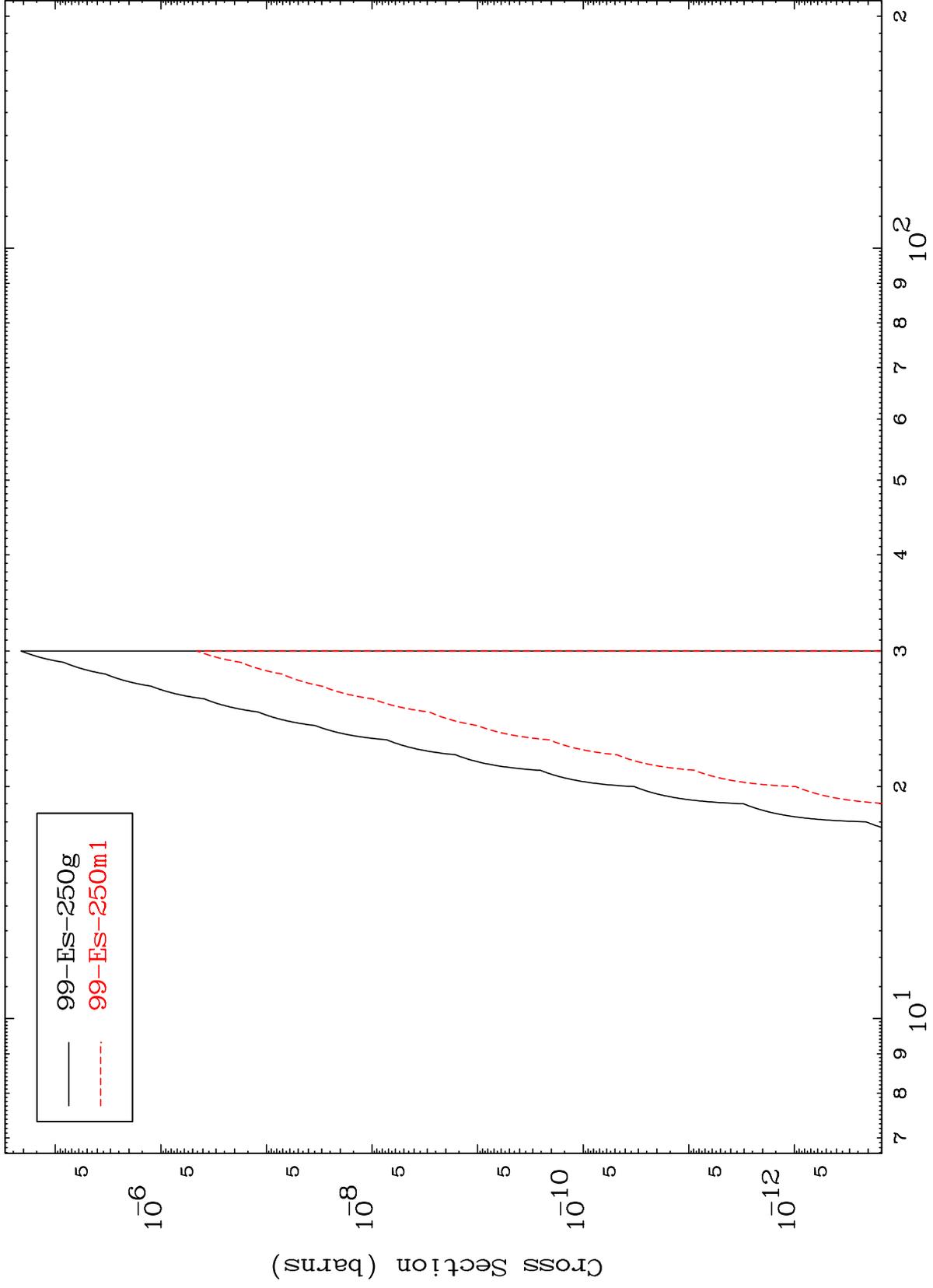
101-Md-253

MAT 9954

(n,n') He-3

101-Md-253

Radionuclide Production Cross Section



30

Incident Energy (MeV)

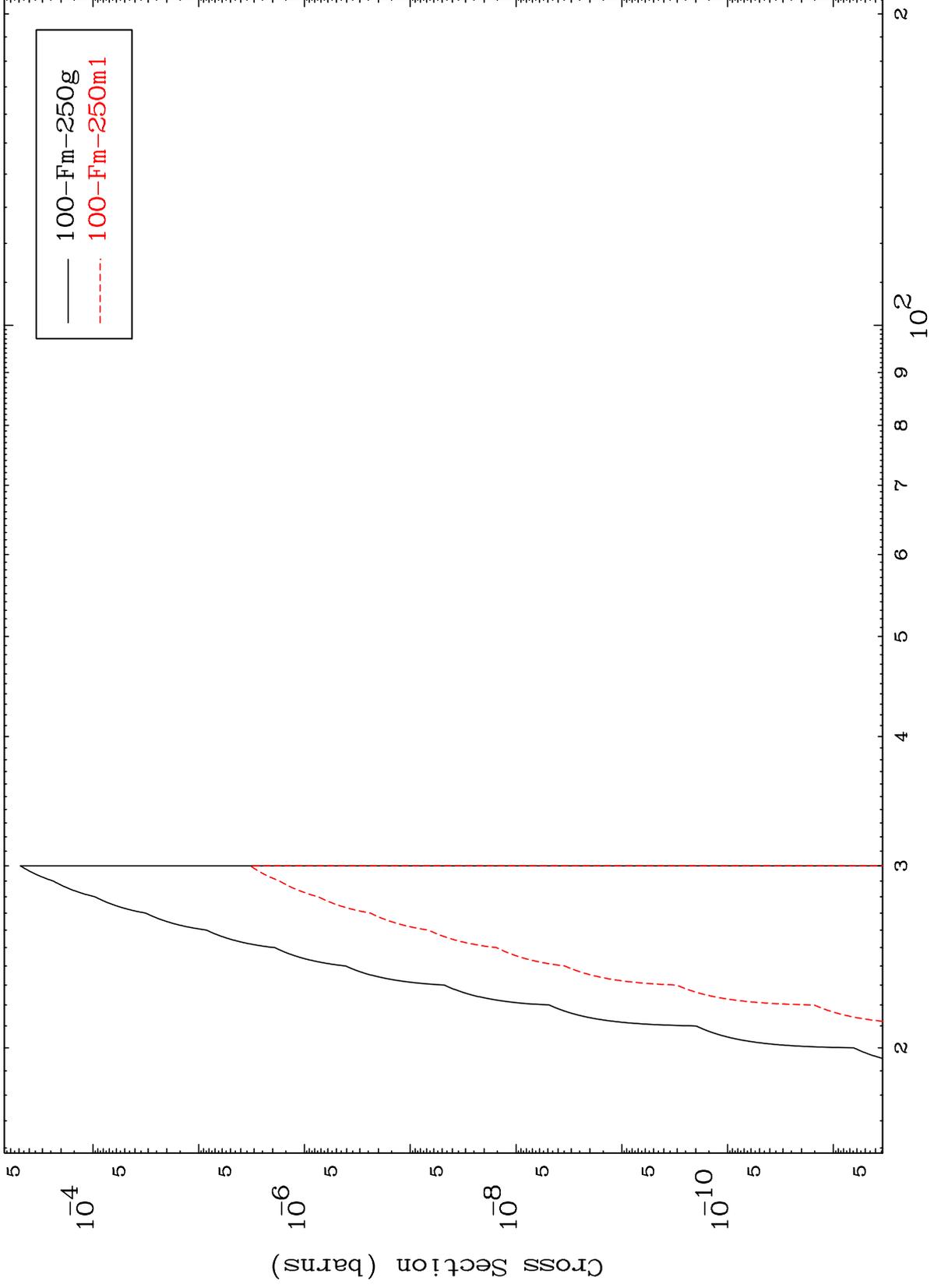
101-Md-253

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(n,3n) p

101-Md-253

Radionuclide Production Cross Section



31

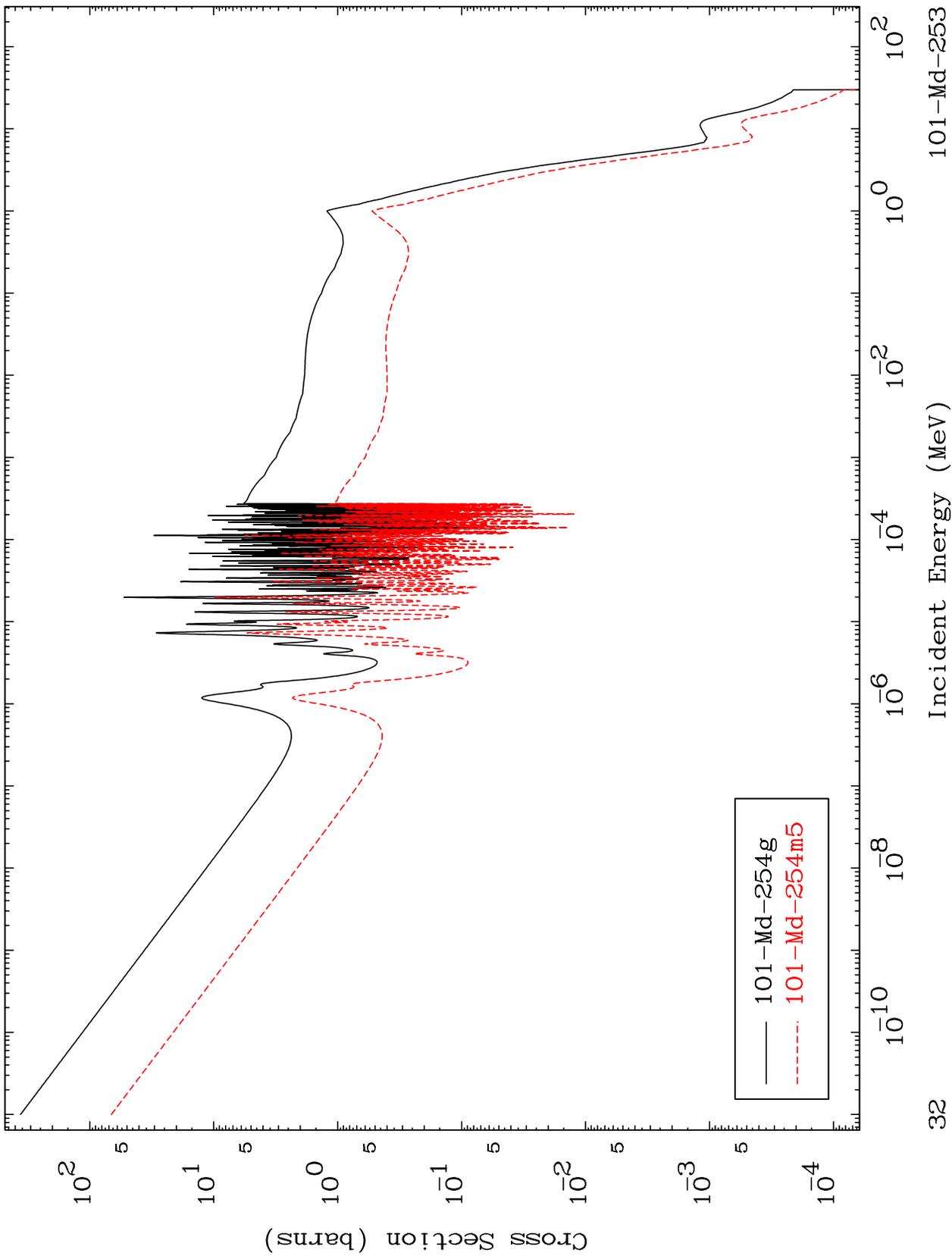
Incident Energy (MeV)

101-Md-253

MAT 9954

101-Md-253

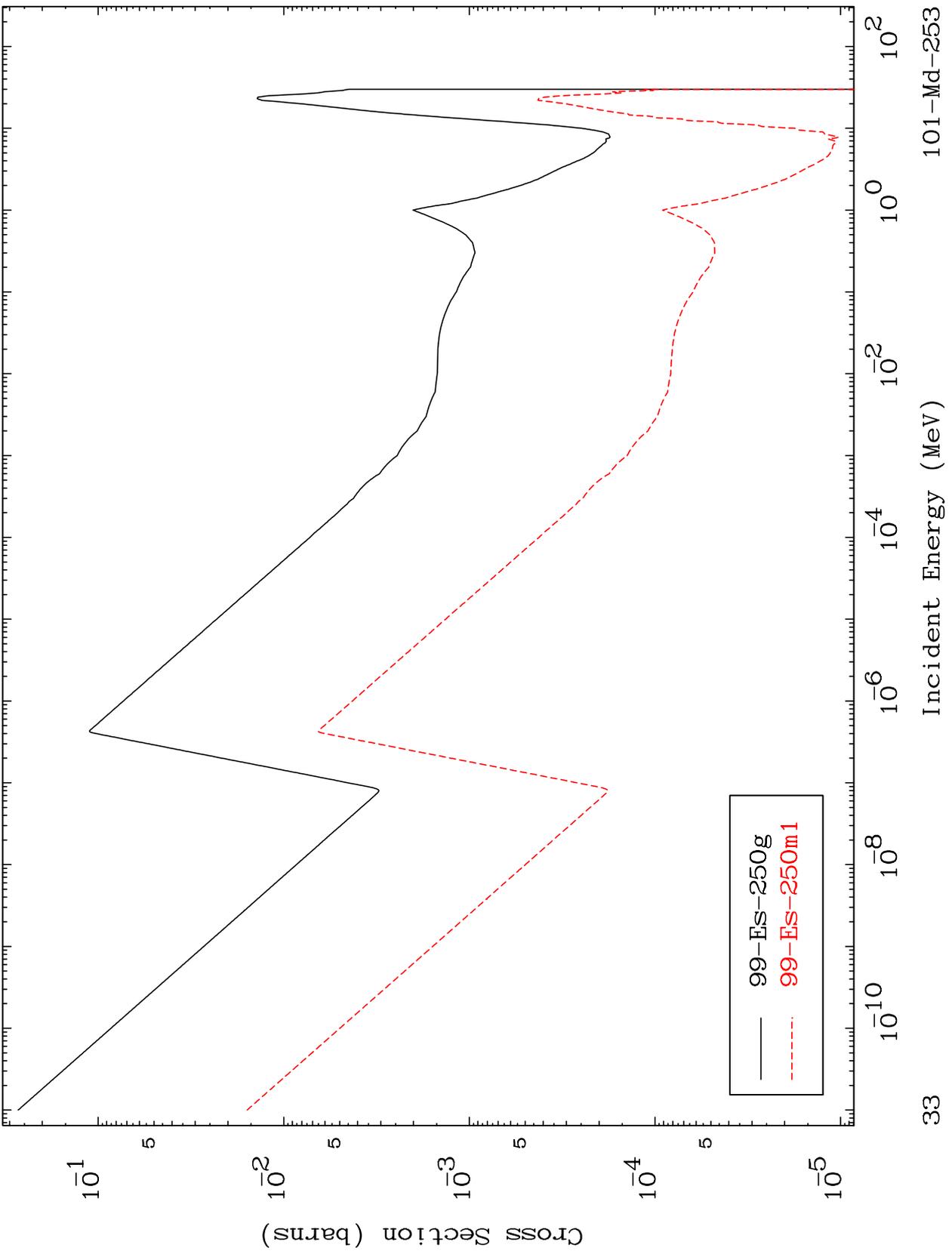
$(n, \gamma)$   
Radionuclide Production Cross Section



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101-Md-253

Radionuclide Production Cross Section  
(n,  $\alpha$ )

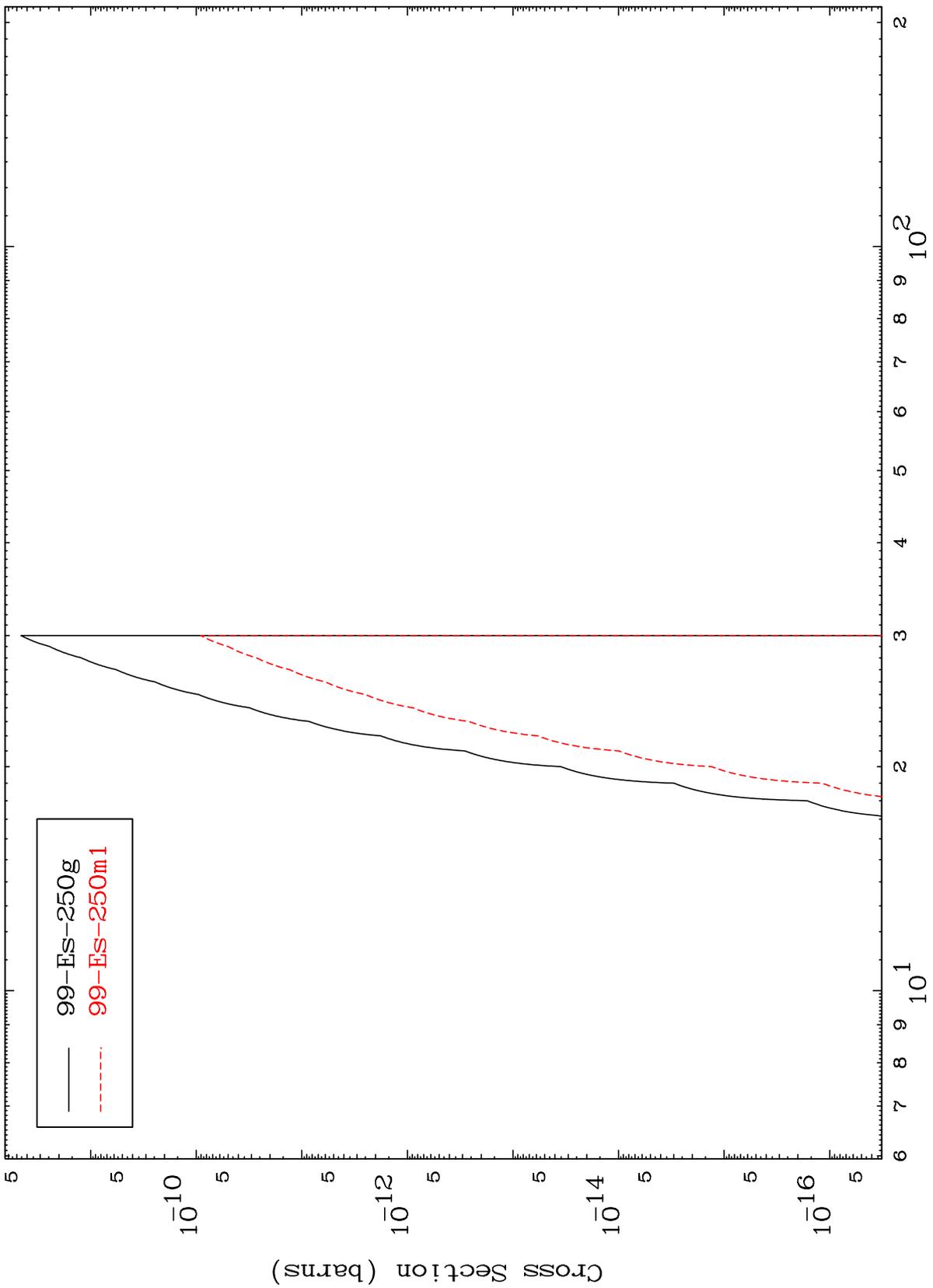


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(n,p) t

101-Md-253

Radionuclide Production Cross Section



99-Es-250g  
99-Es-250m1

34

Incident Energy (MeV)

101-Md-253