

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

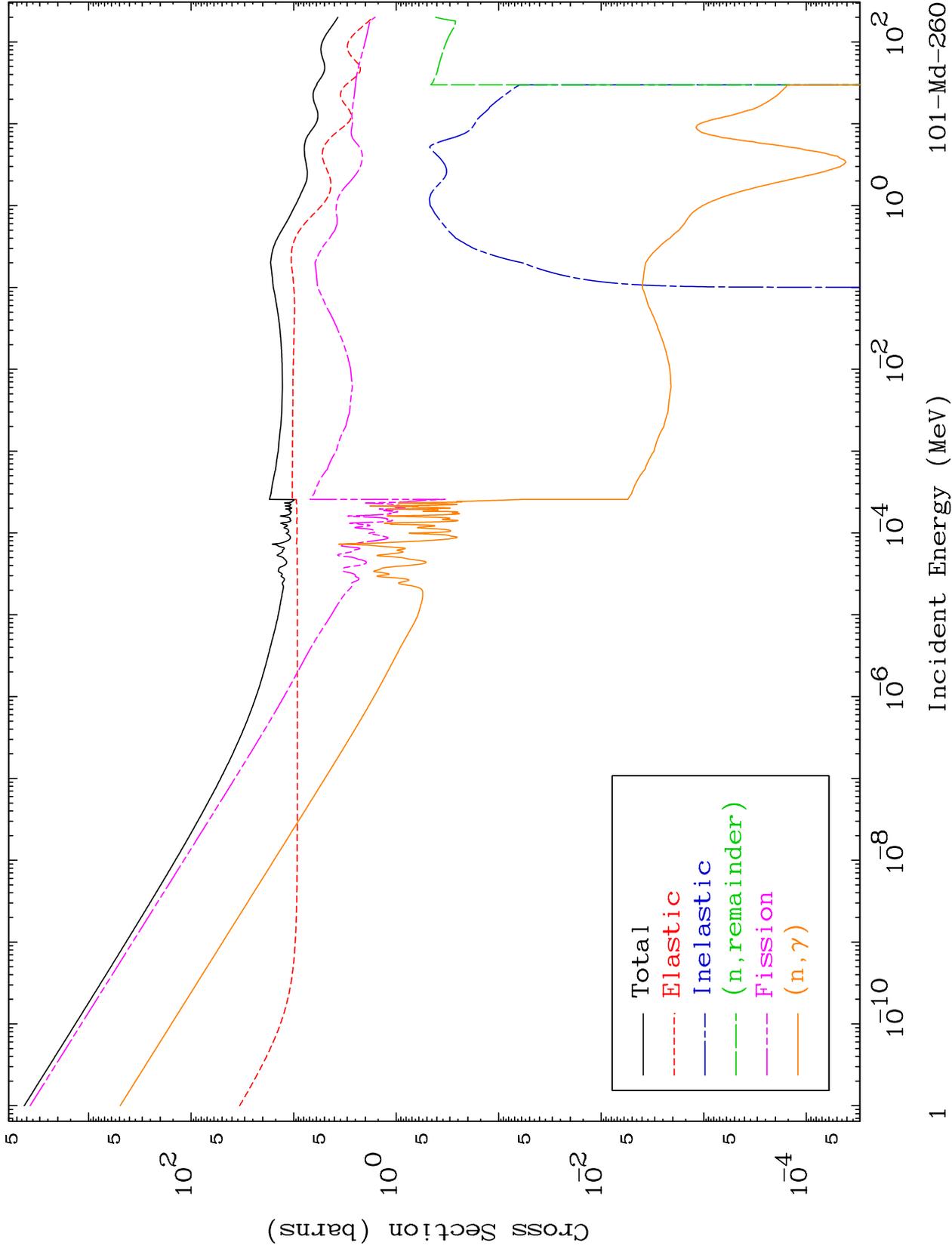
Press Mouse Button to Start

MAT 9961

Major

293 Kelvin Cross Sections

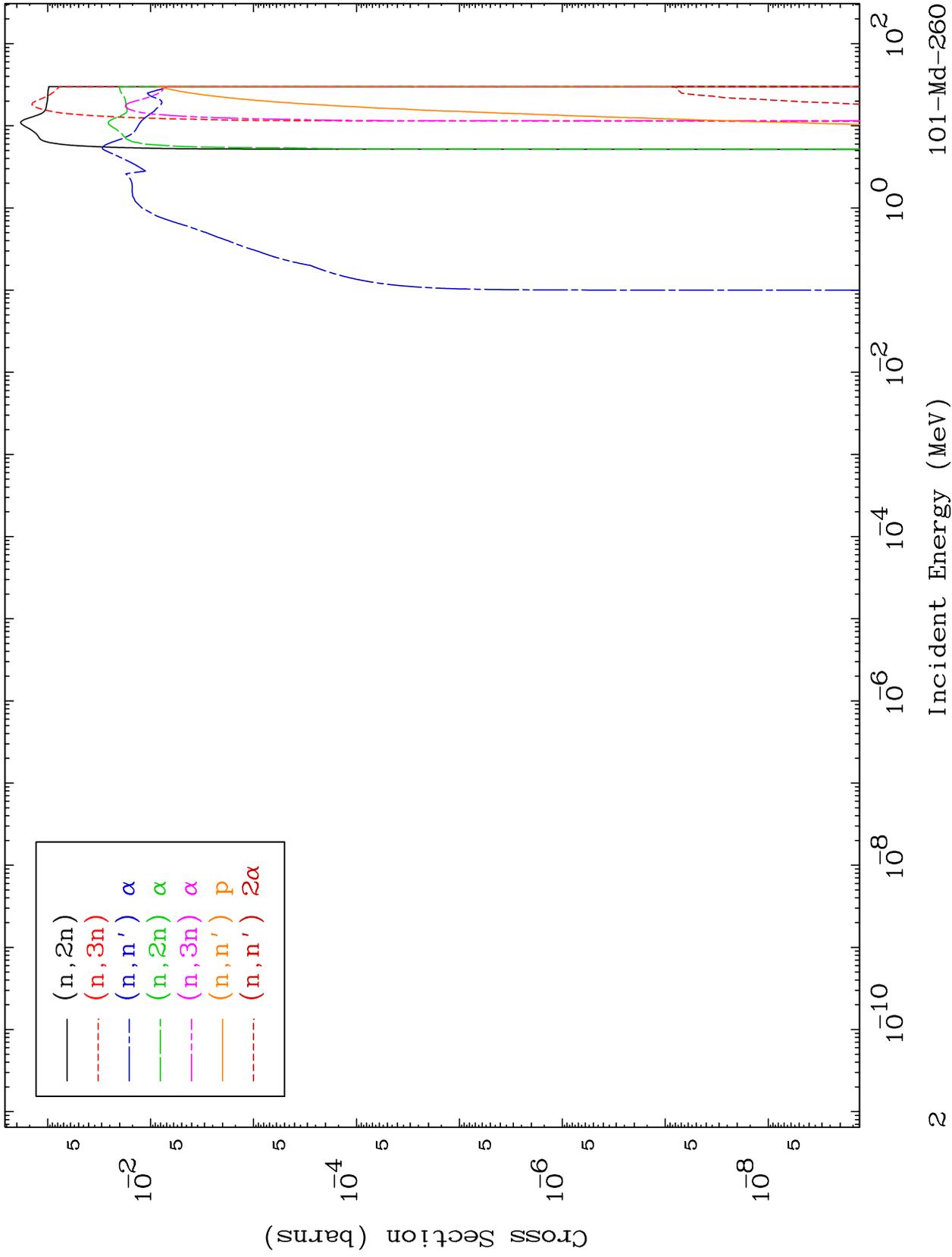
101-Md-260



MAT 9961

Neutron Production
293 Kelvin Cross Sections

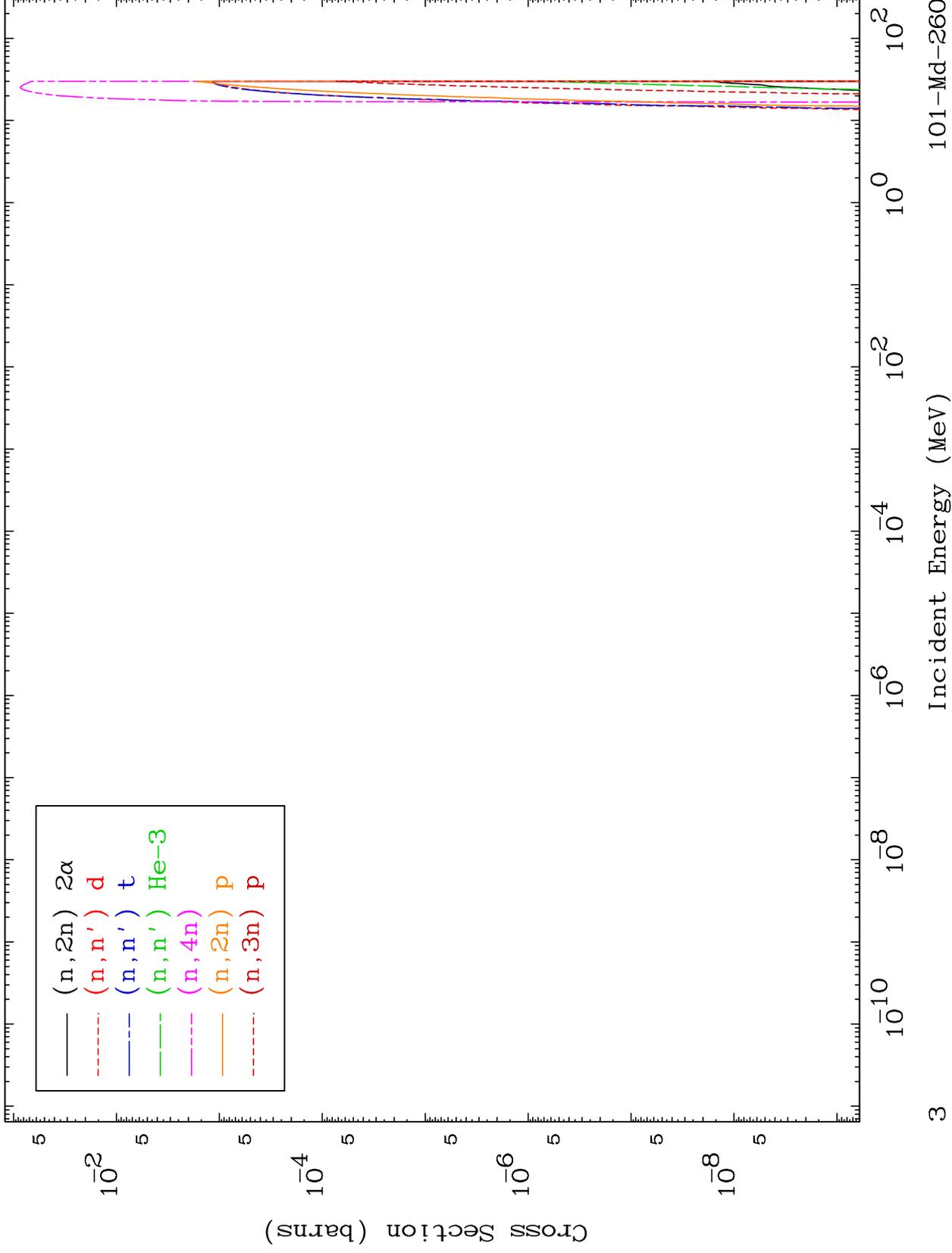
101-Md-260



MAT 9961

Neutron Production
293 Kelvin Cross Sections

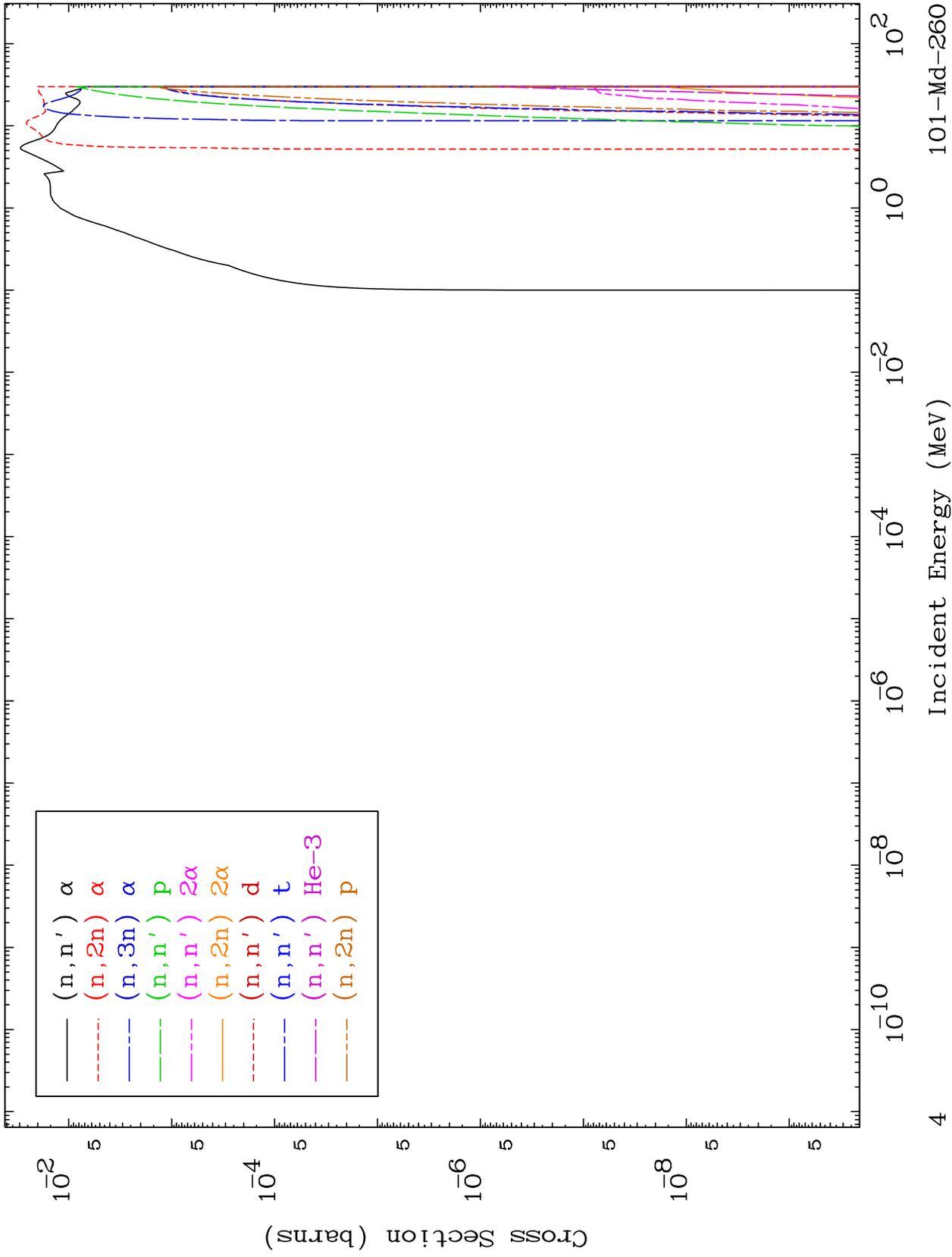
101-Md-260



MAT 9961

Charged Particle
293 Kelvin Cross Sections

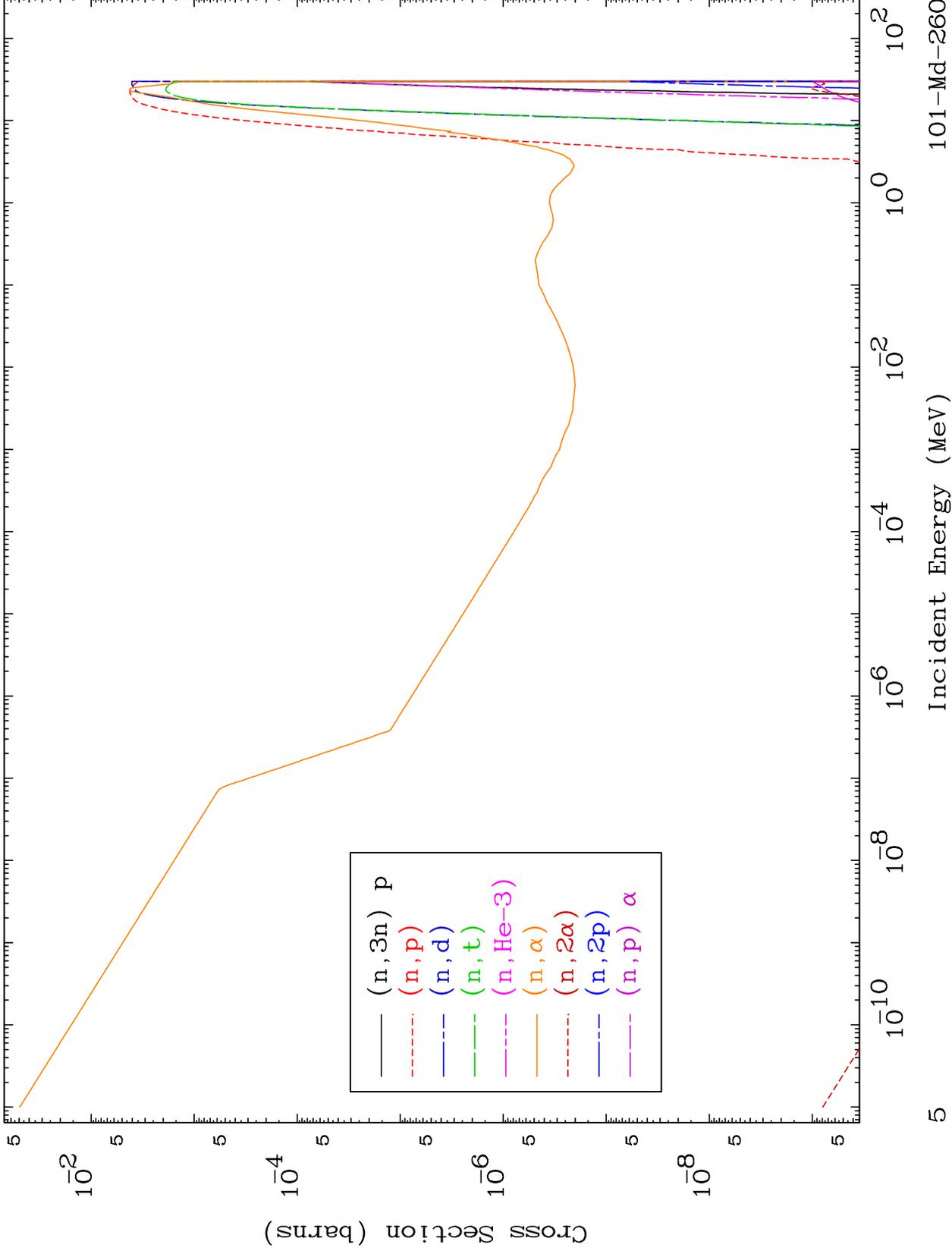
101-Md-260



MAT 9961

Charged Particle
293 Kelvin Cross Sections

101-Md-260

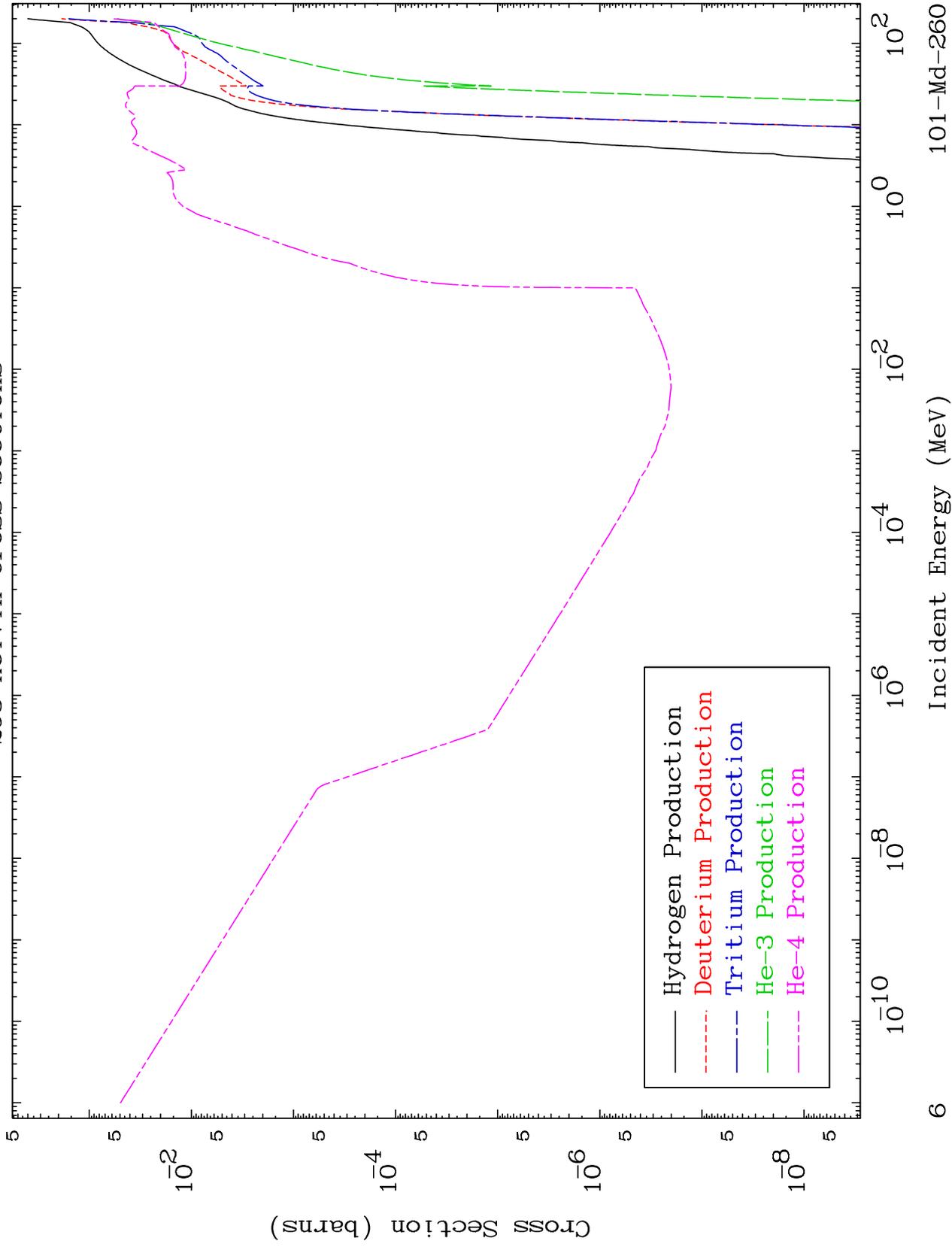


5

MAT 9961

Particle Production
293 Kelvin Cross Sections

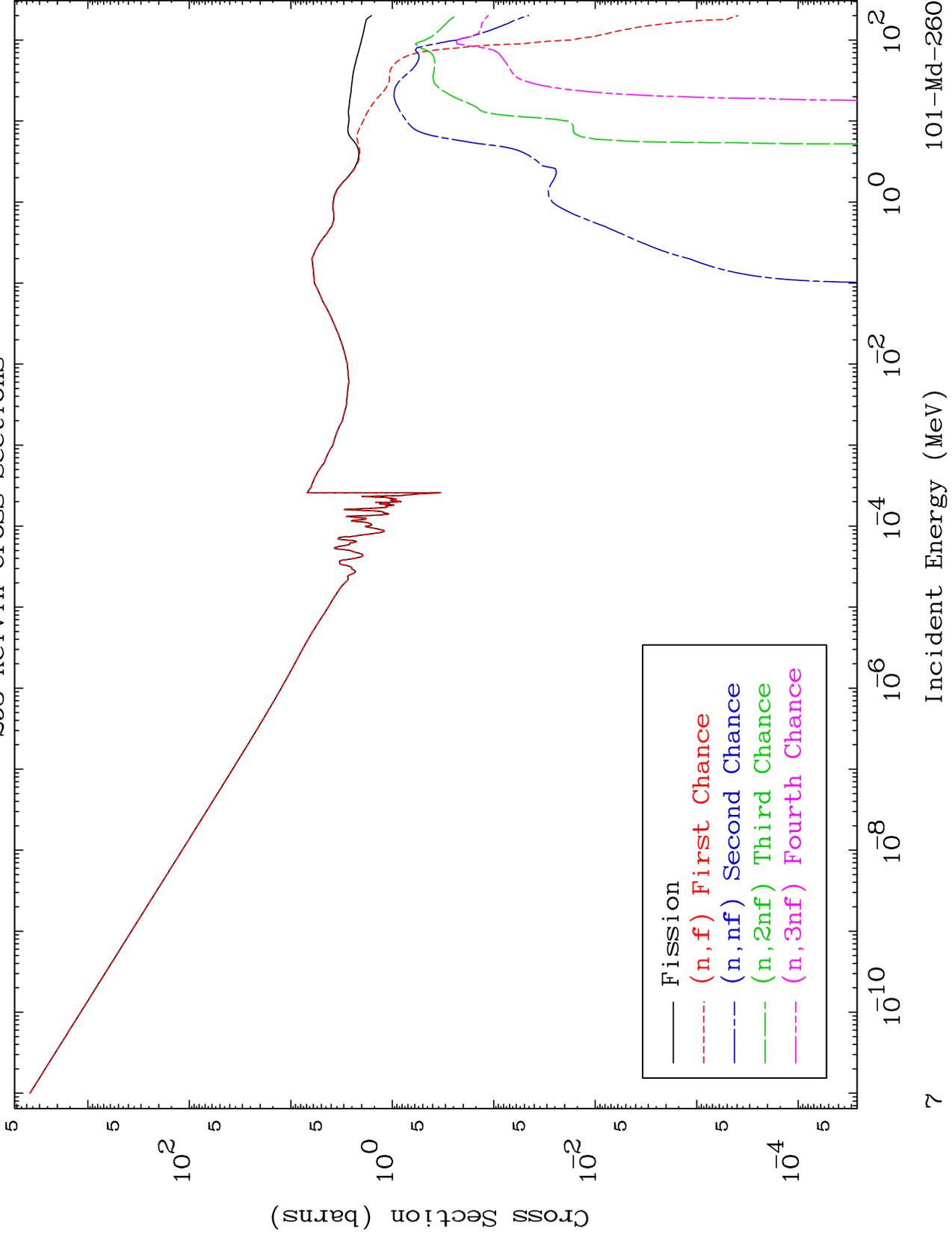
101-Md-260



MAT 9961

Fission
293 Kelvin Cross Sections

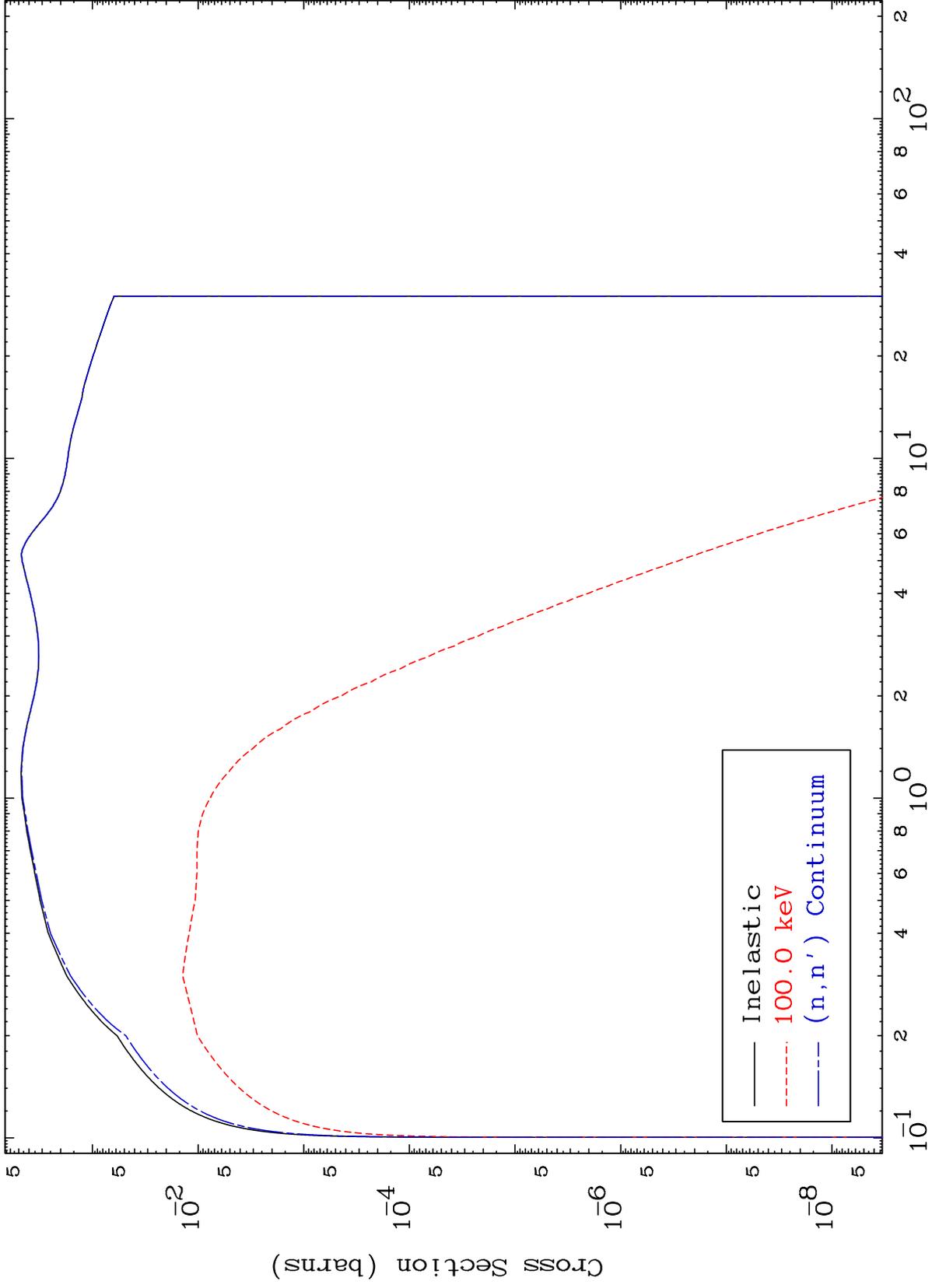
101-Md-260



MAT 9961

(n,n') Level
293 Kelvin Cross Sections

101-Md-260



Incident Energy (MeV)

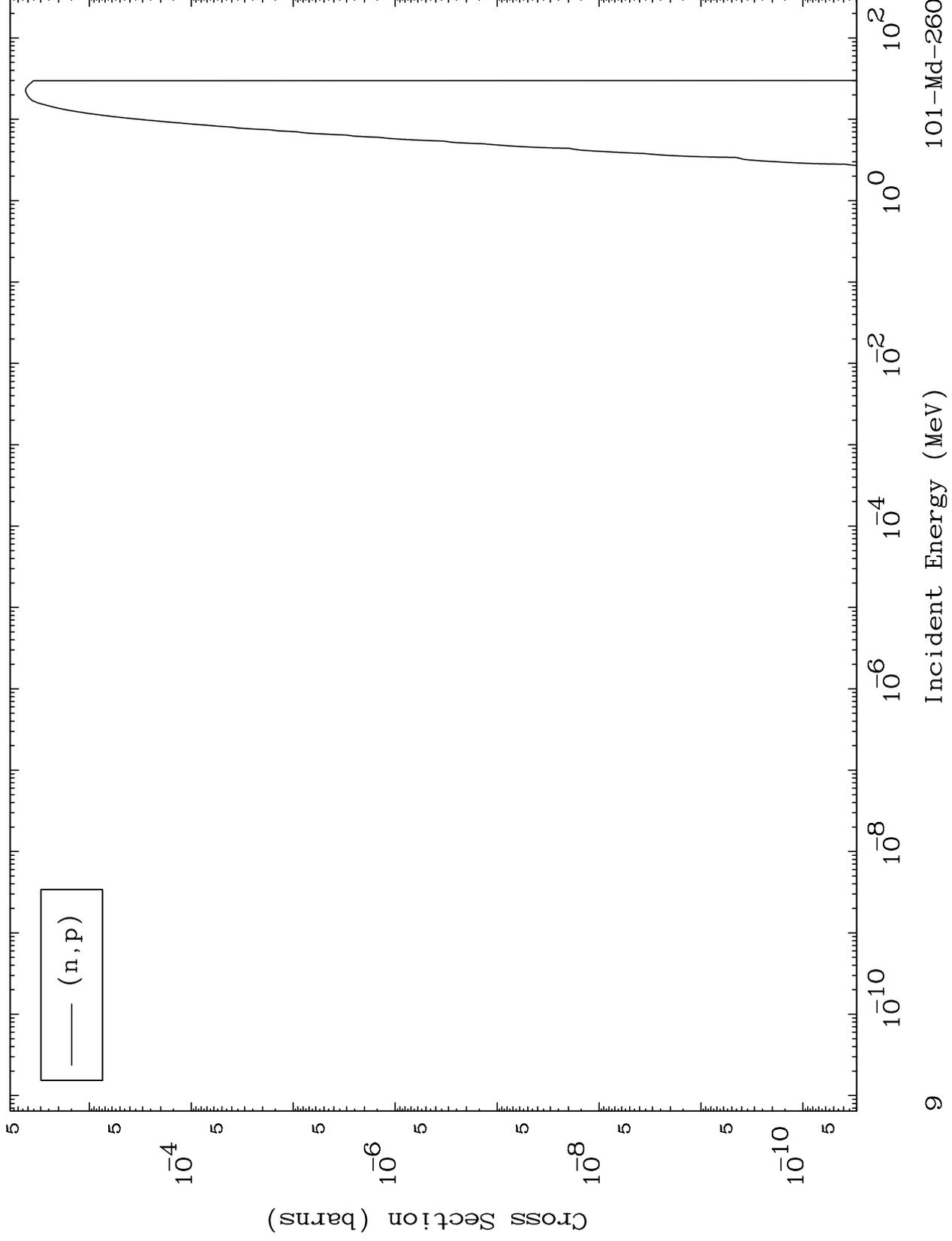
101-Md-260

8

MAT 9961

(n,p) Levels
293 Kelvin Cross Sections

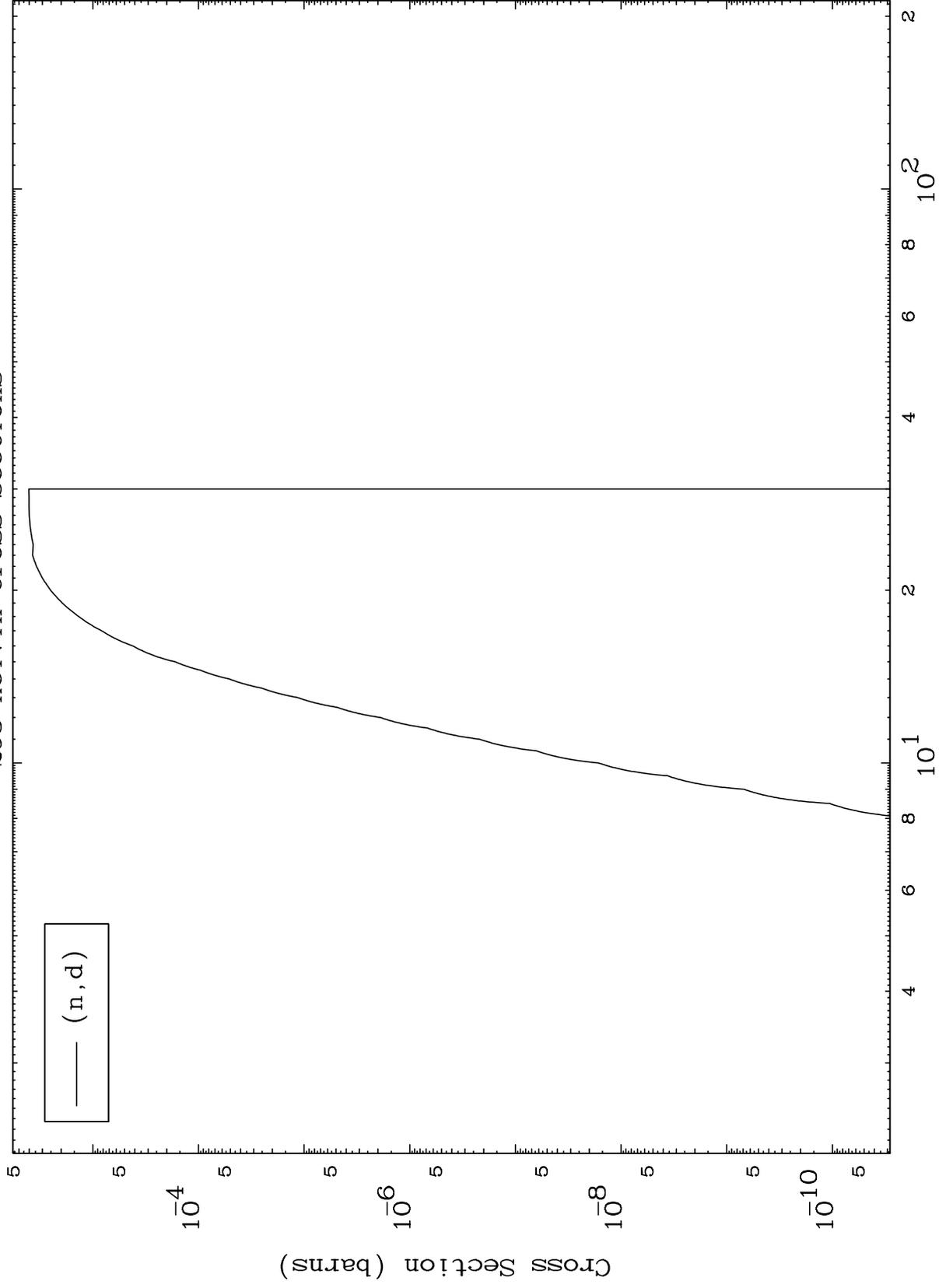
101-Md-260



MAT 9961

(n,d) Levels
293 Kelvin Cross Sections

101-Md-260



10

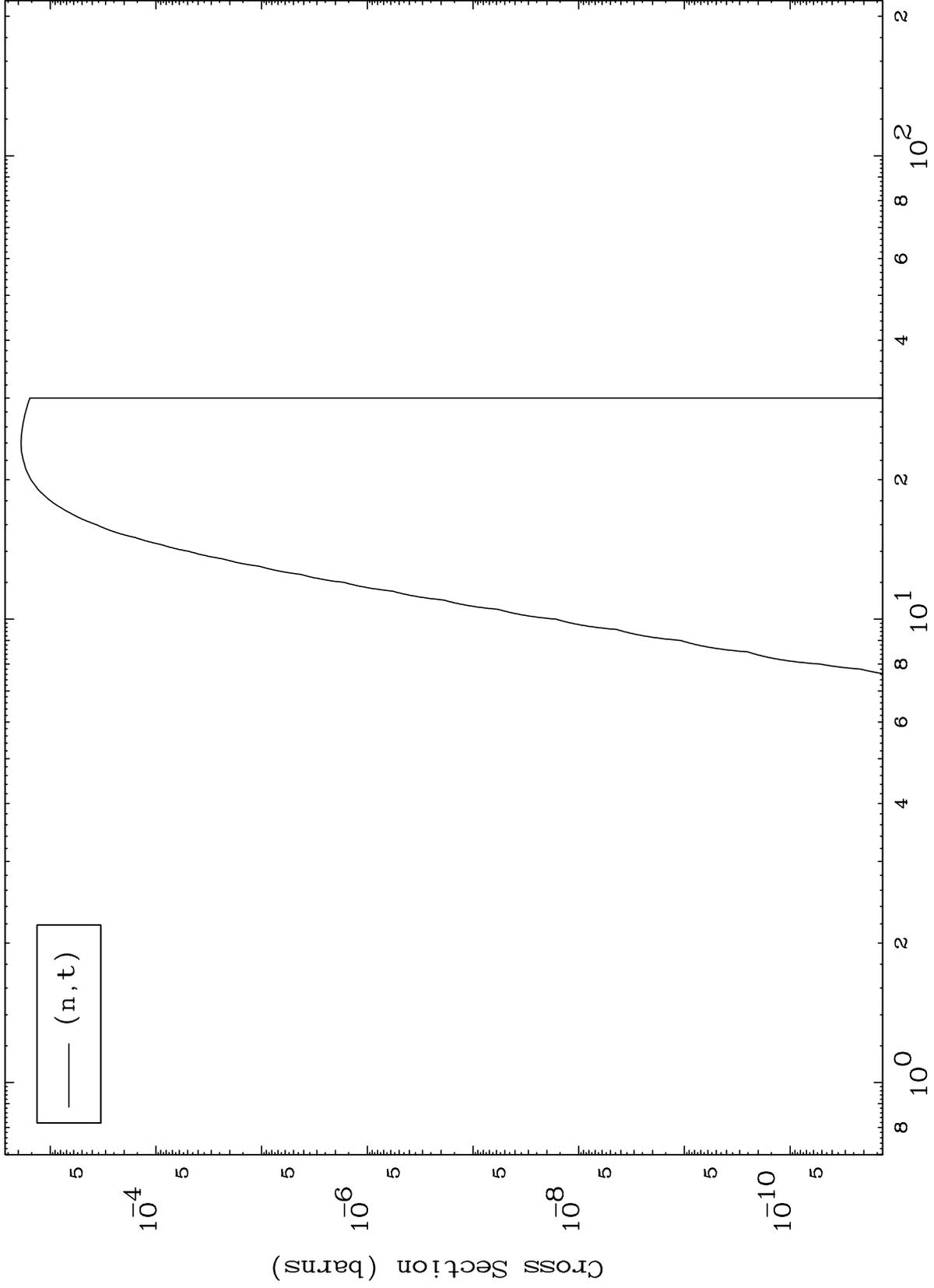
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,t) Levels
293 Kelvin Cross Sections

101-Md-260



11

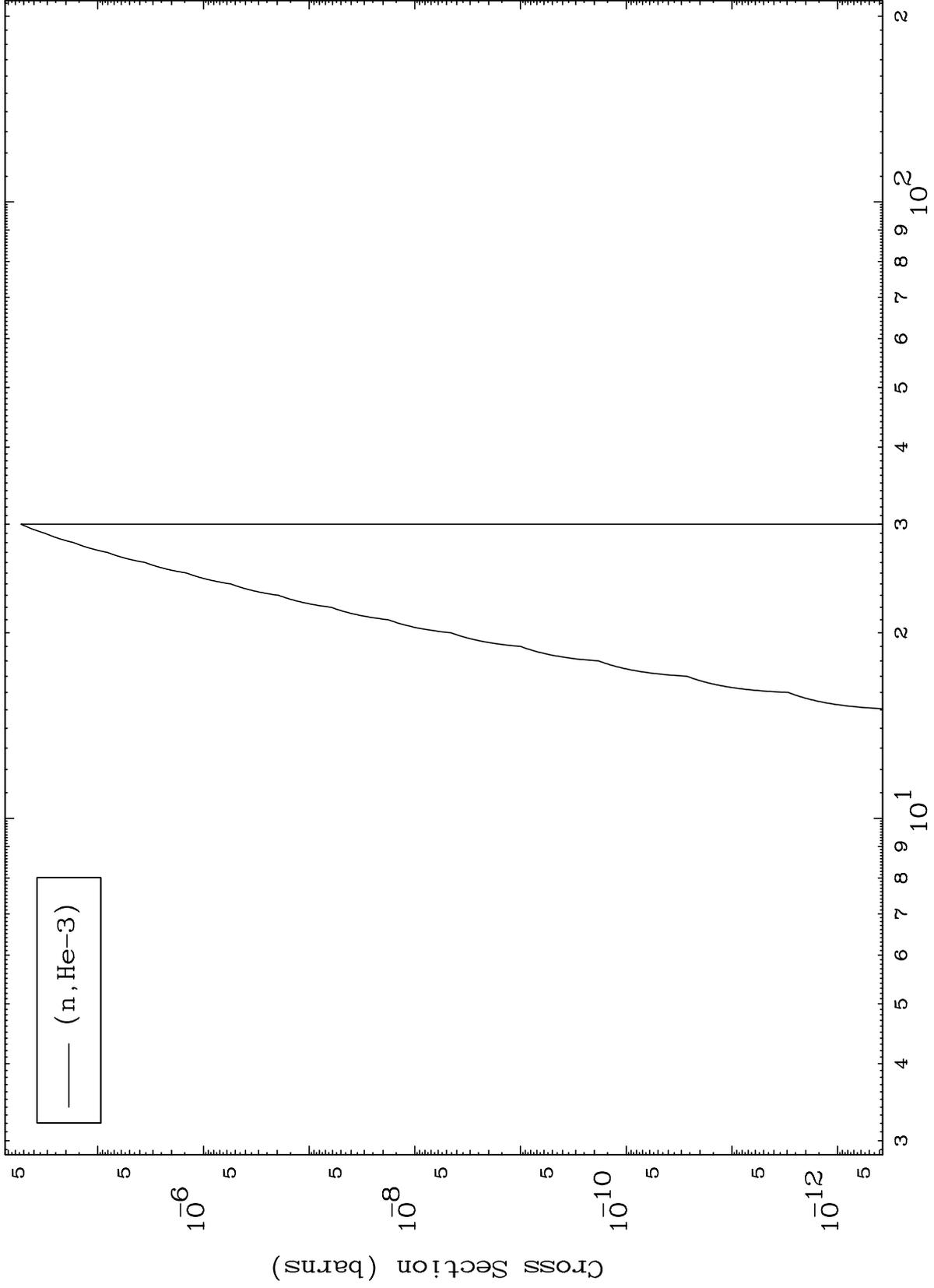
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-260



12

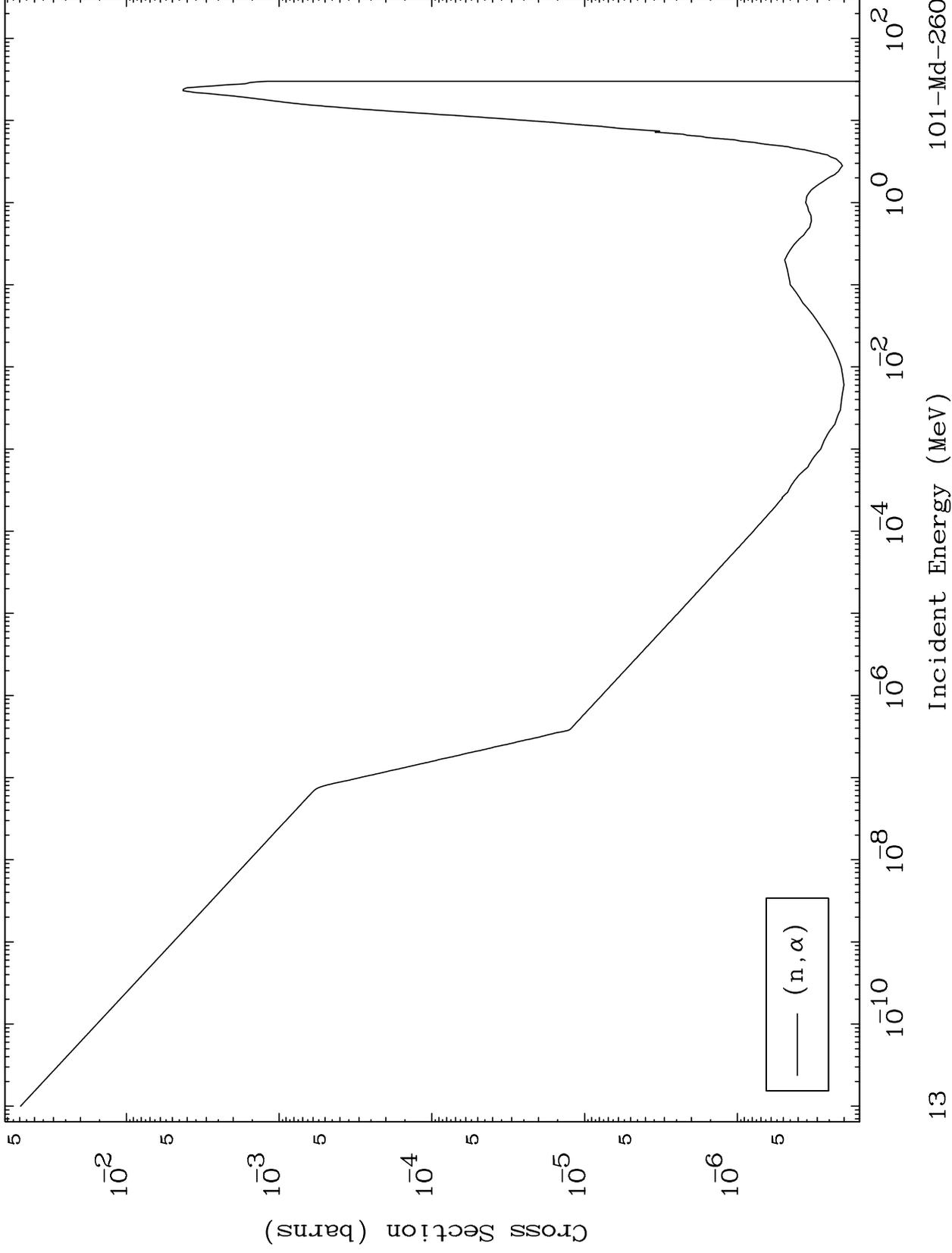
Incident Energy (MeV)

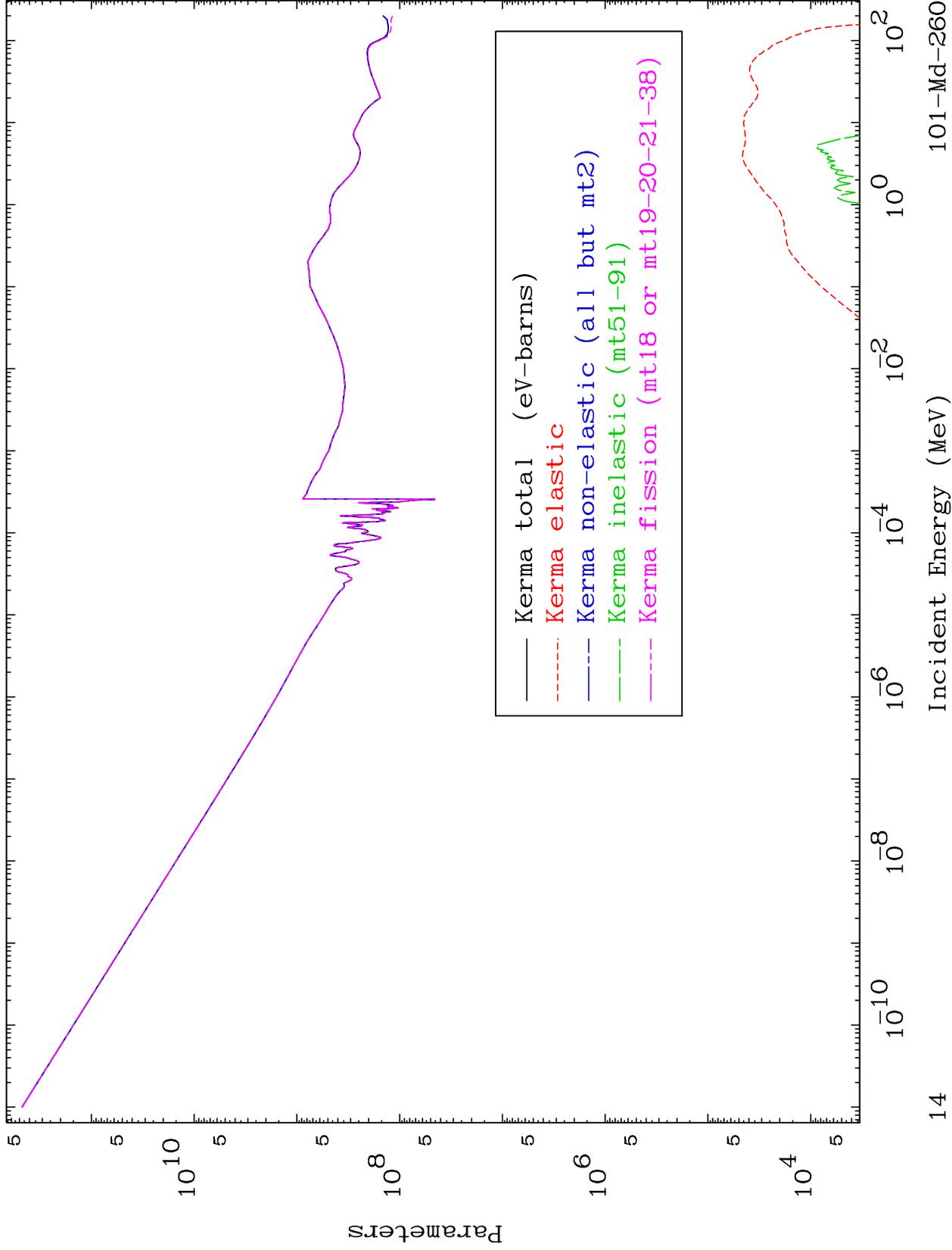
101-Md-260

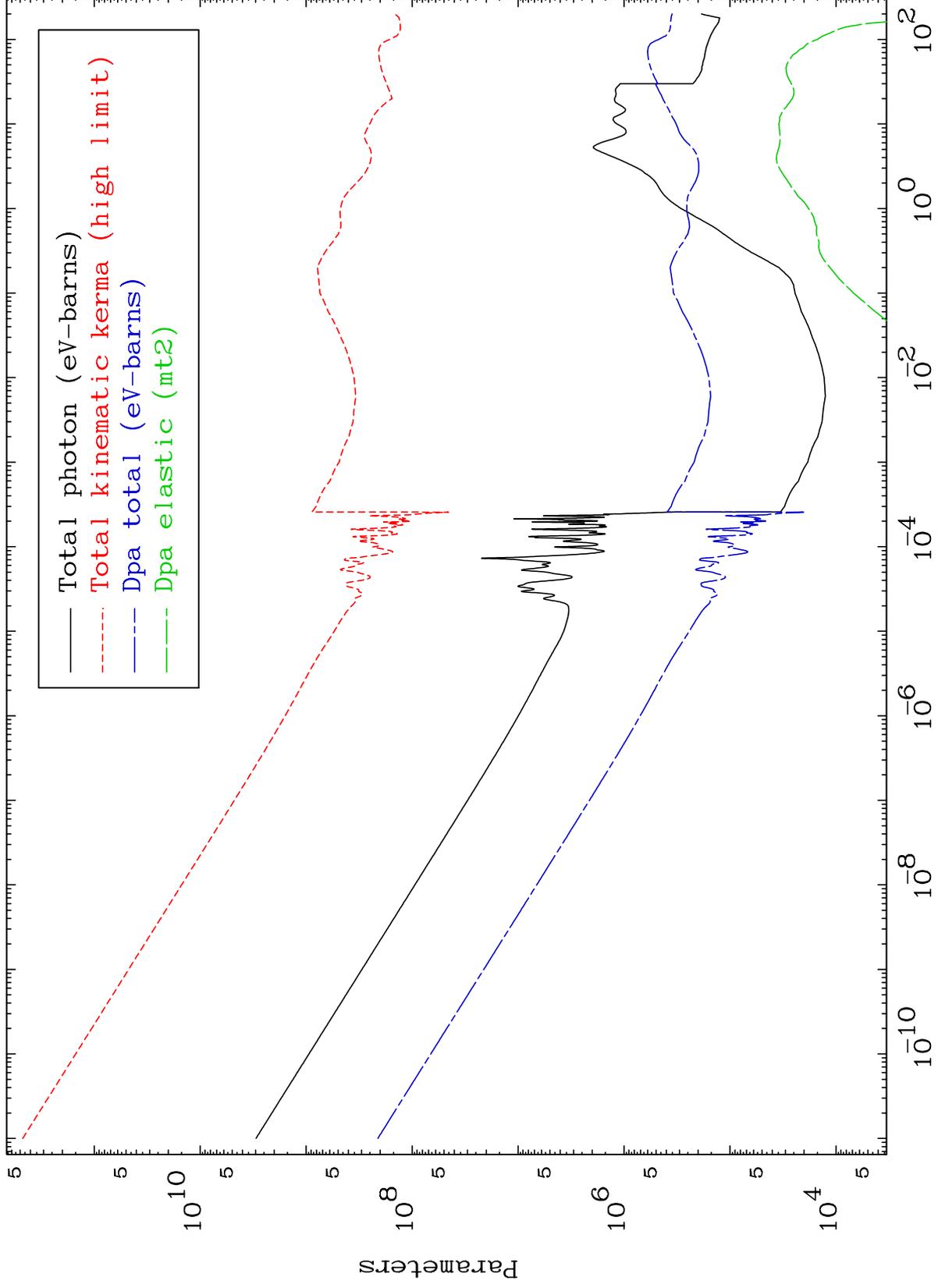
MAT 9961

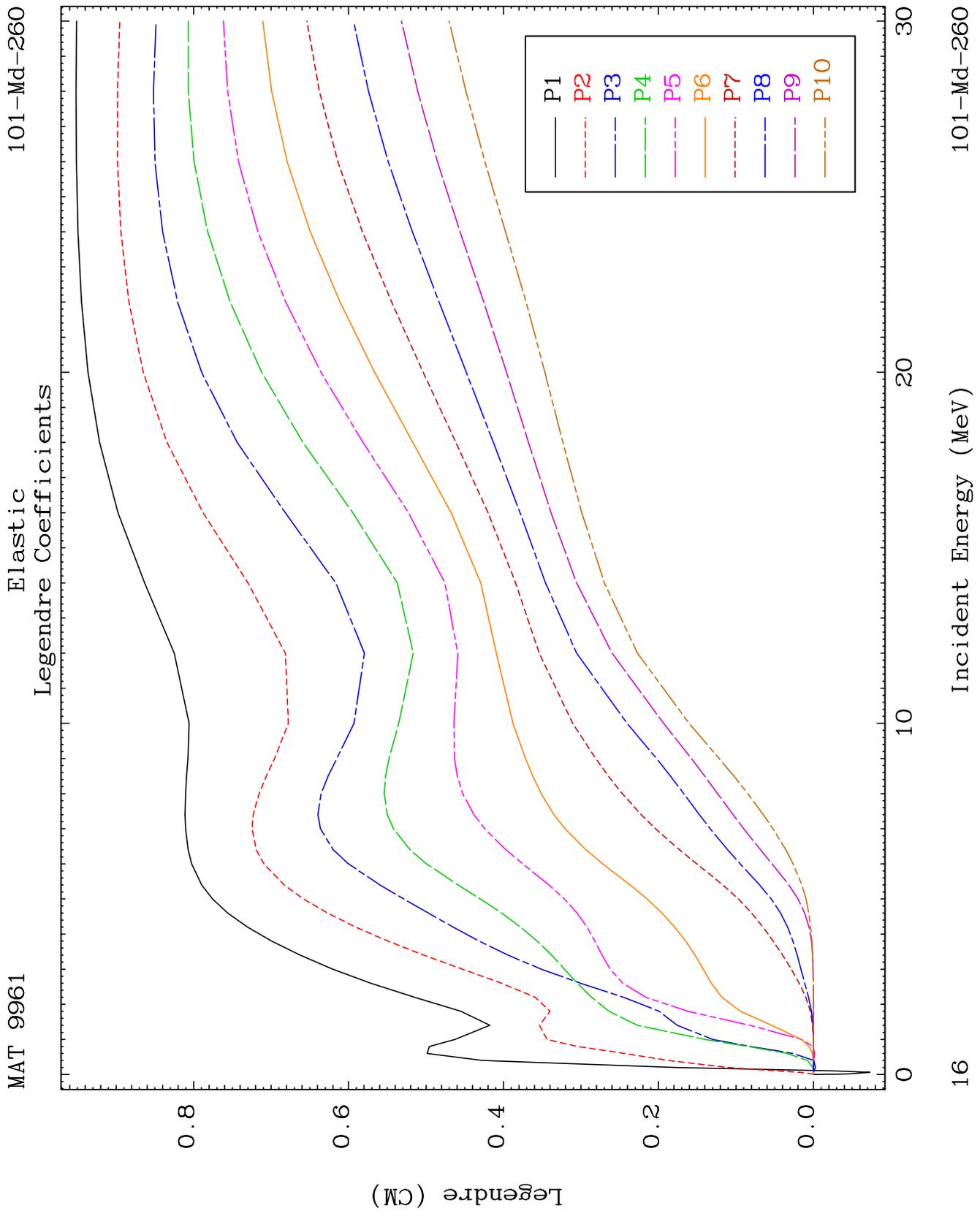
(n, α) Levels
293 Kelvin Cross Sections

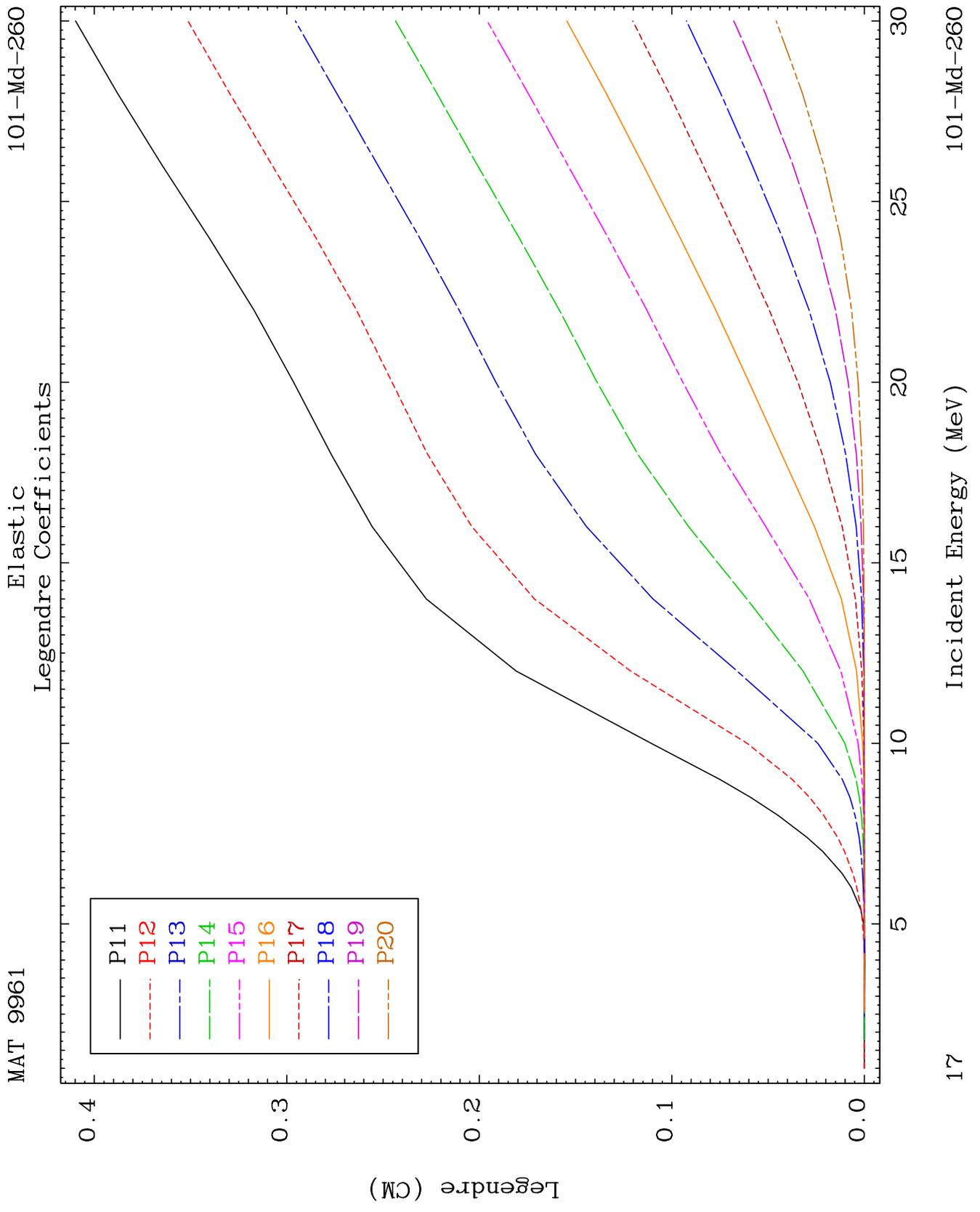
101-Md-260







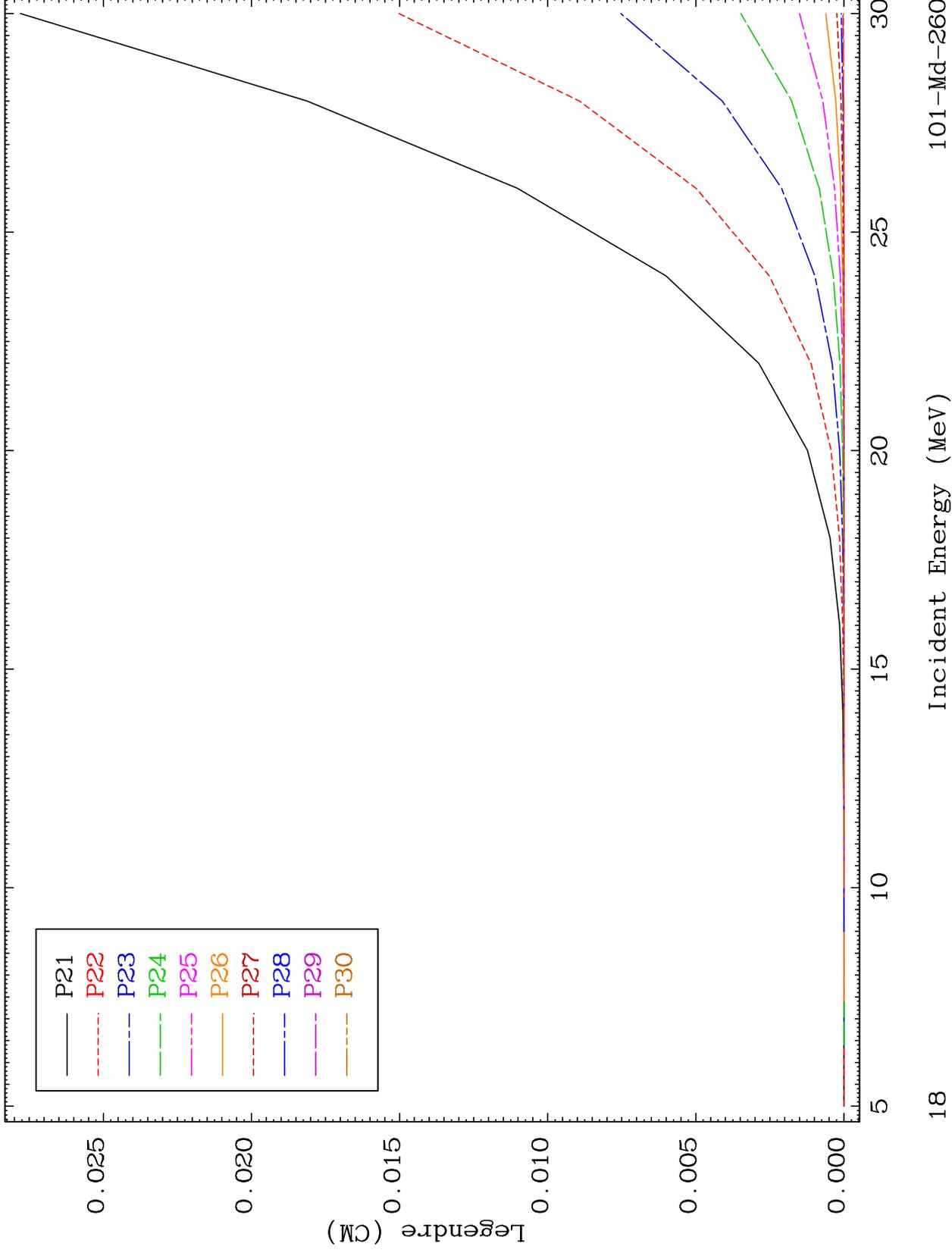




MAT 9961

Elastic Legendre Coefficients

101-Md-260



18

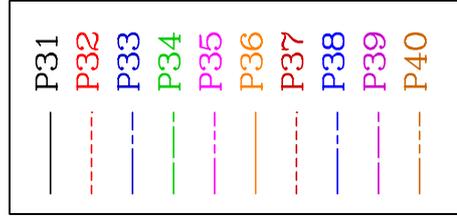
Incident Energy (MeV)

101-Md-260

MAT 9961

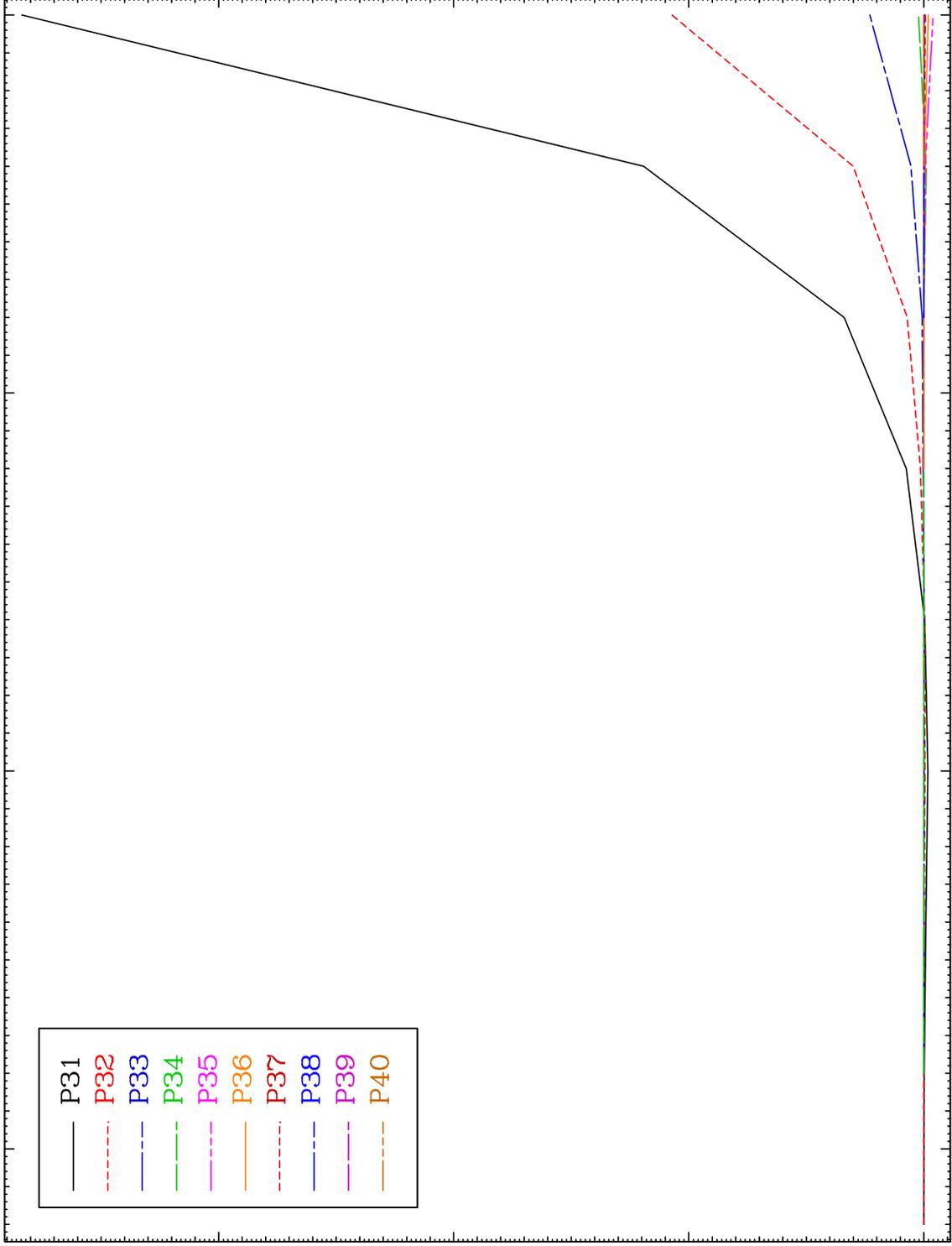
Elastic Legendre Coefficients

101-Md-260



$\times 10^{-6}$

Legendre (CM)



30

25

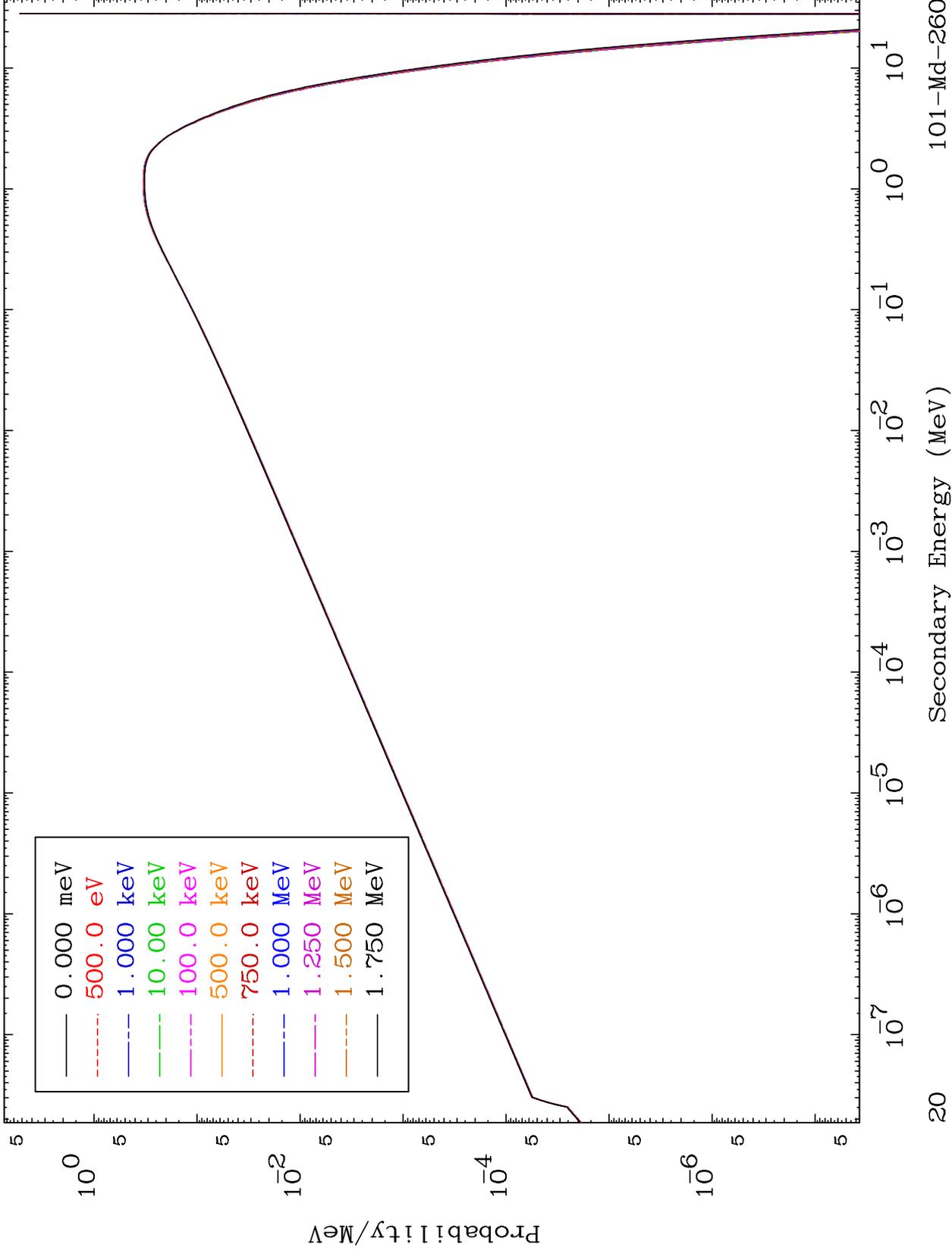
20

15

19

Incident Energy (MeV)

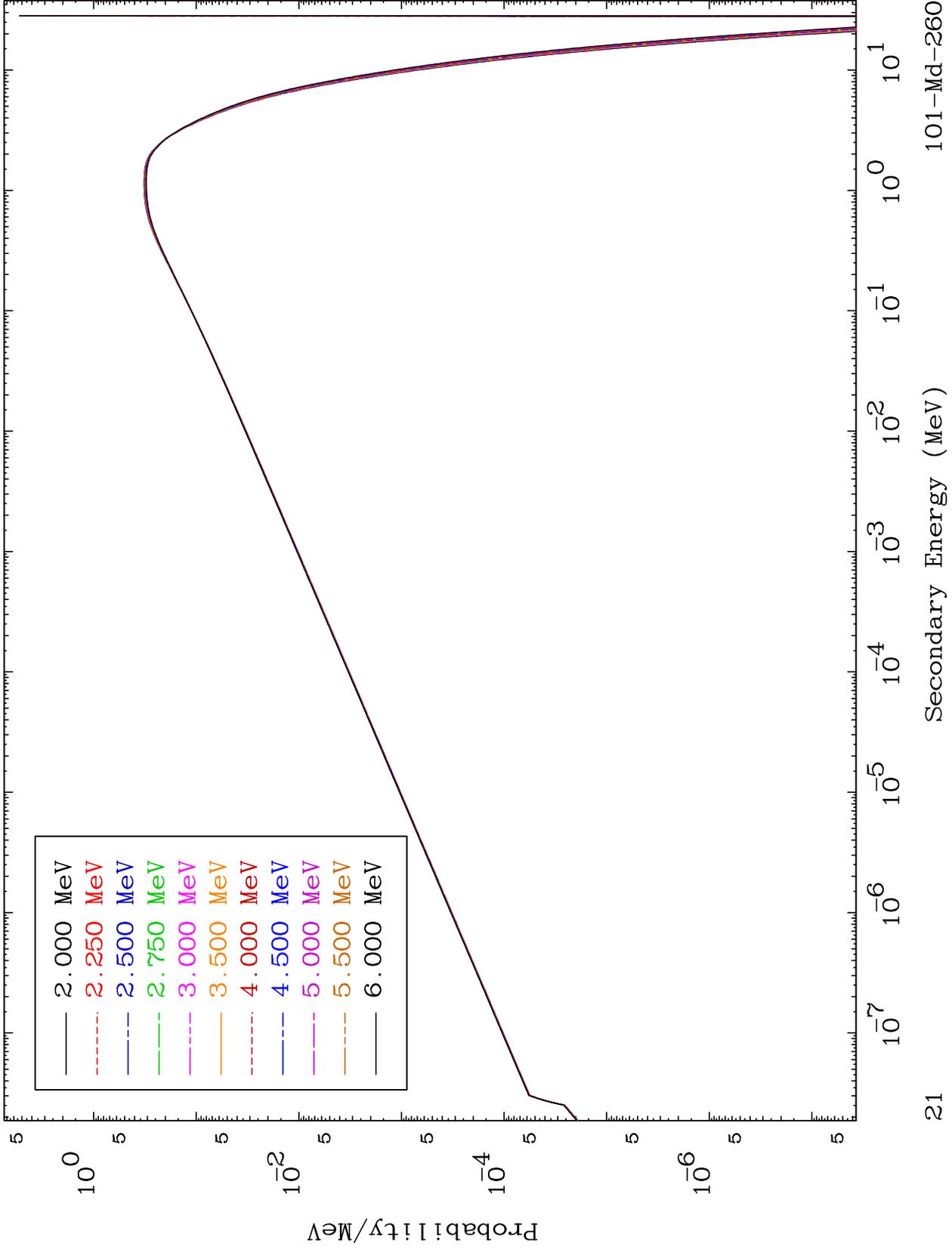
101-Md-260



MAT 9961

Fission Energy Distribution

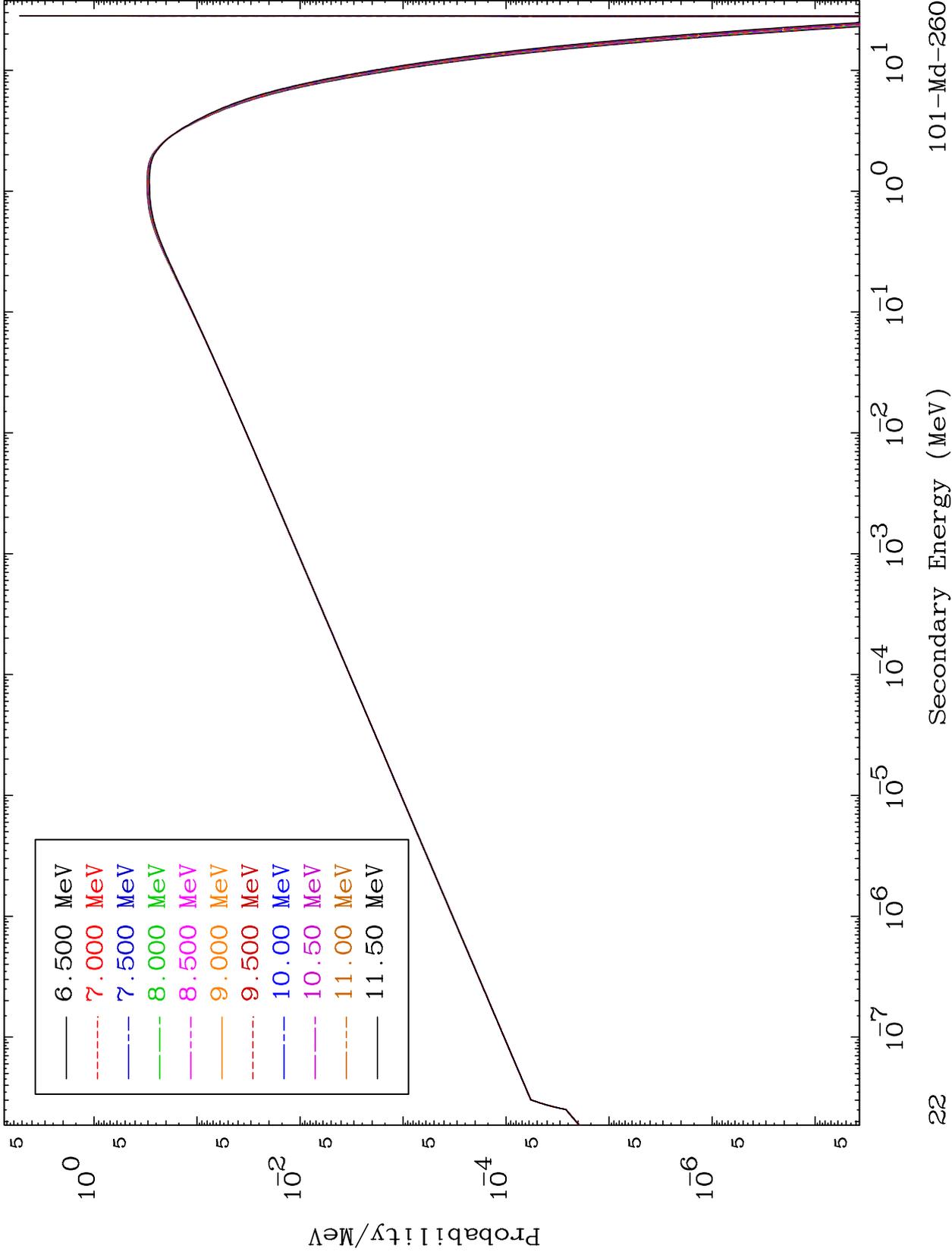
101-Md-260



MAT 9961

Fission Energy Distribution

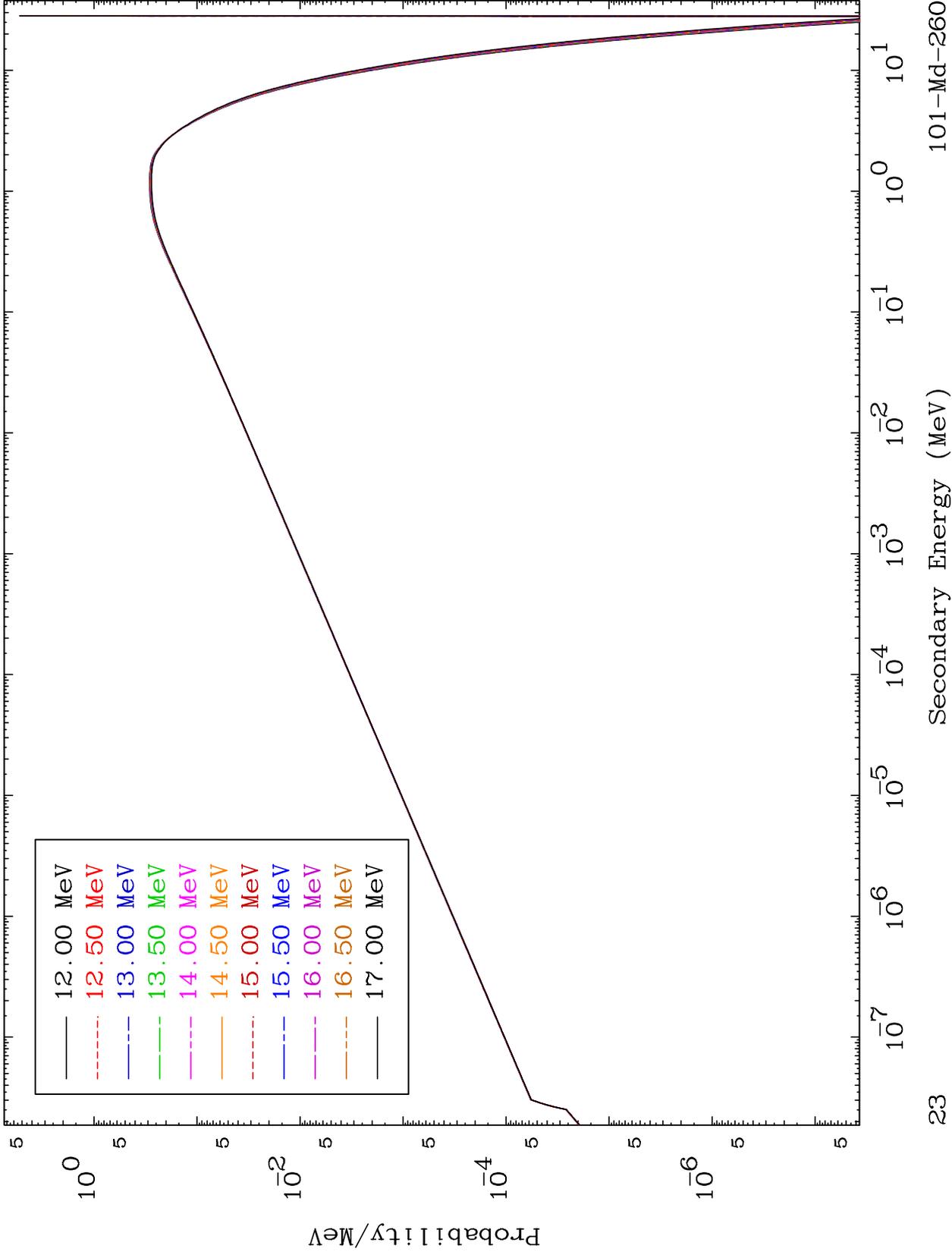
101-Md-260



MAT 9961

Fission Energy Distribution

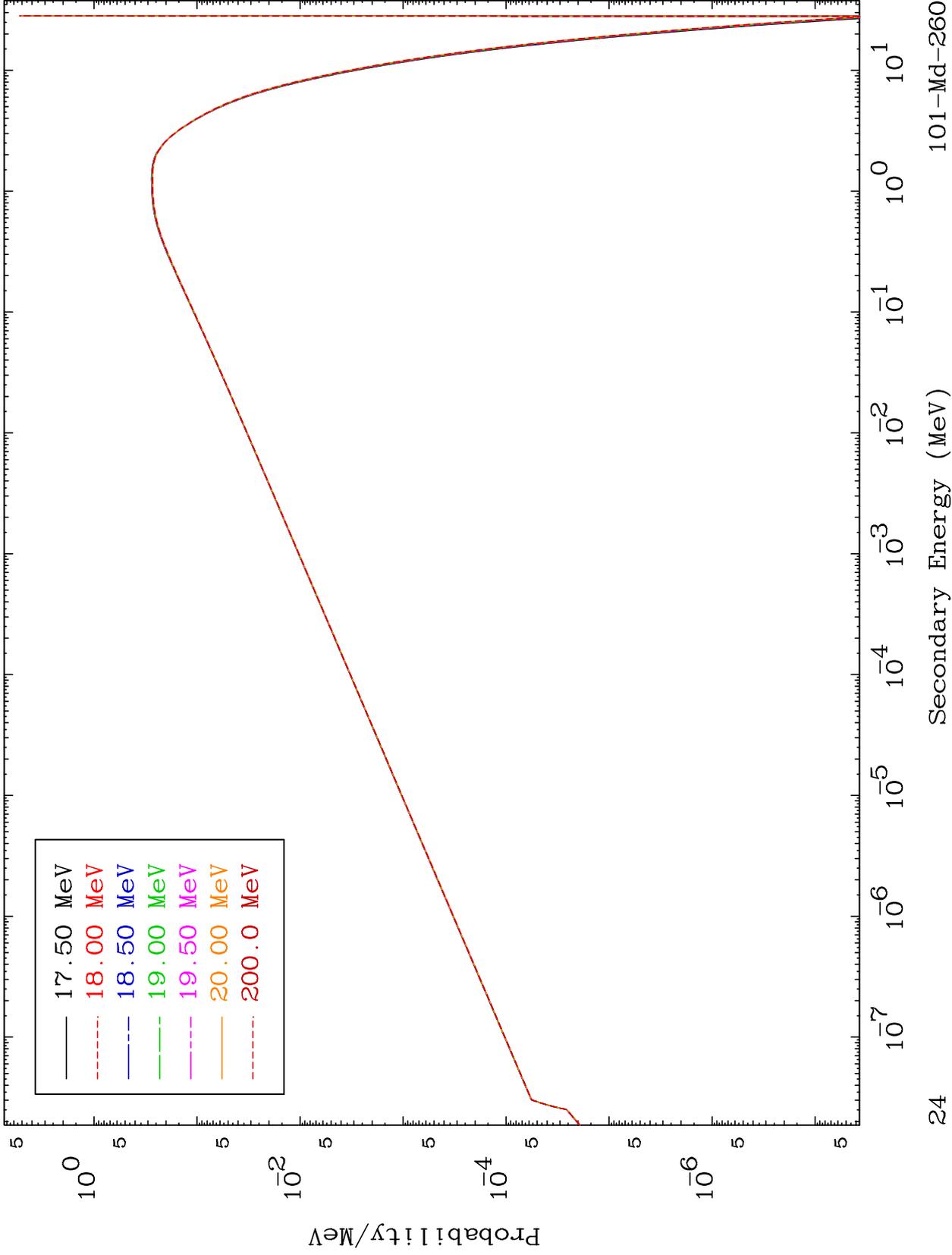
101-Md-260



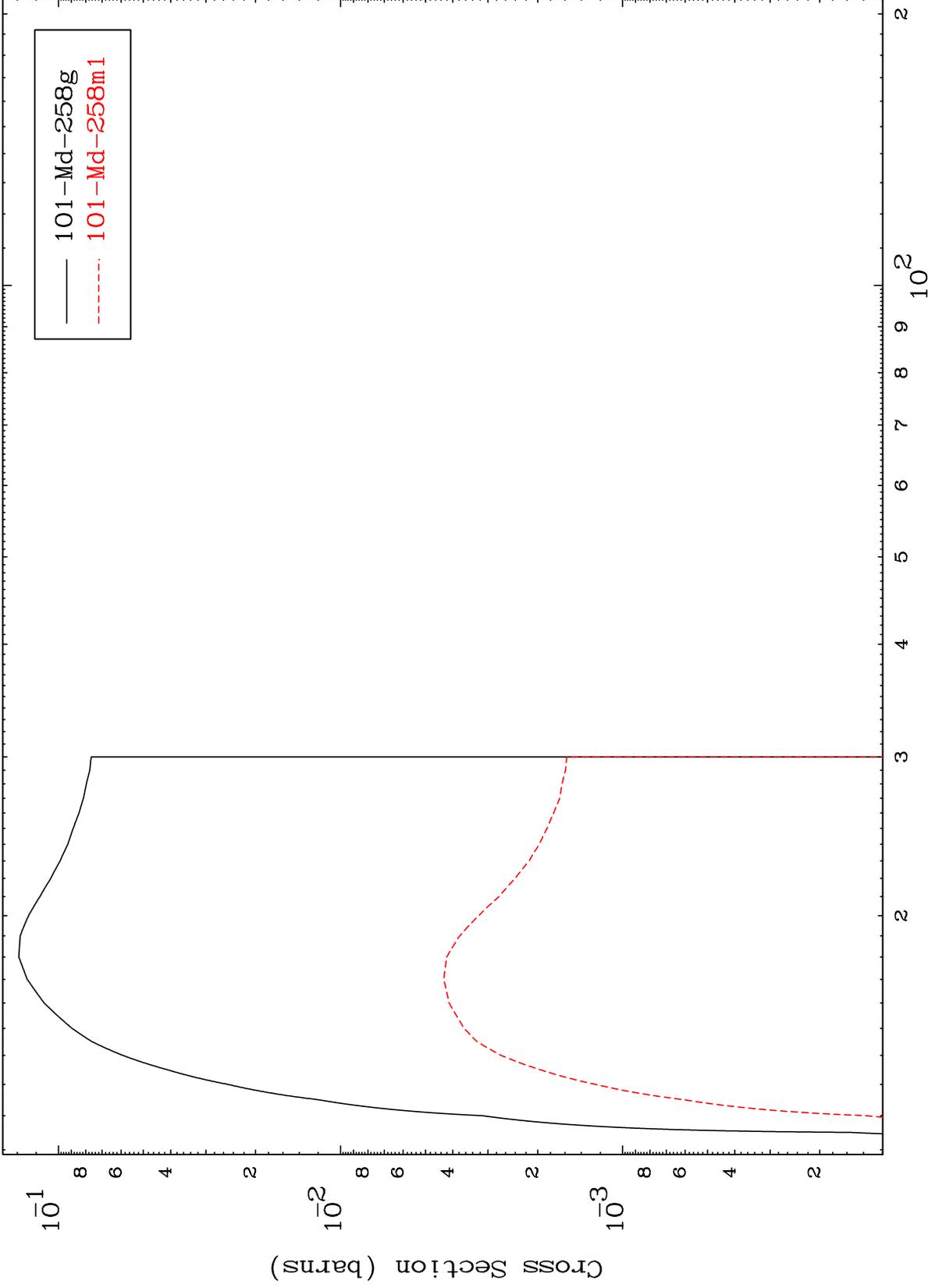
MAT 9961

Fission
Energy Distribution

101-Md-260



Radionuclide Production Cross Section

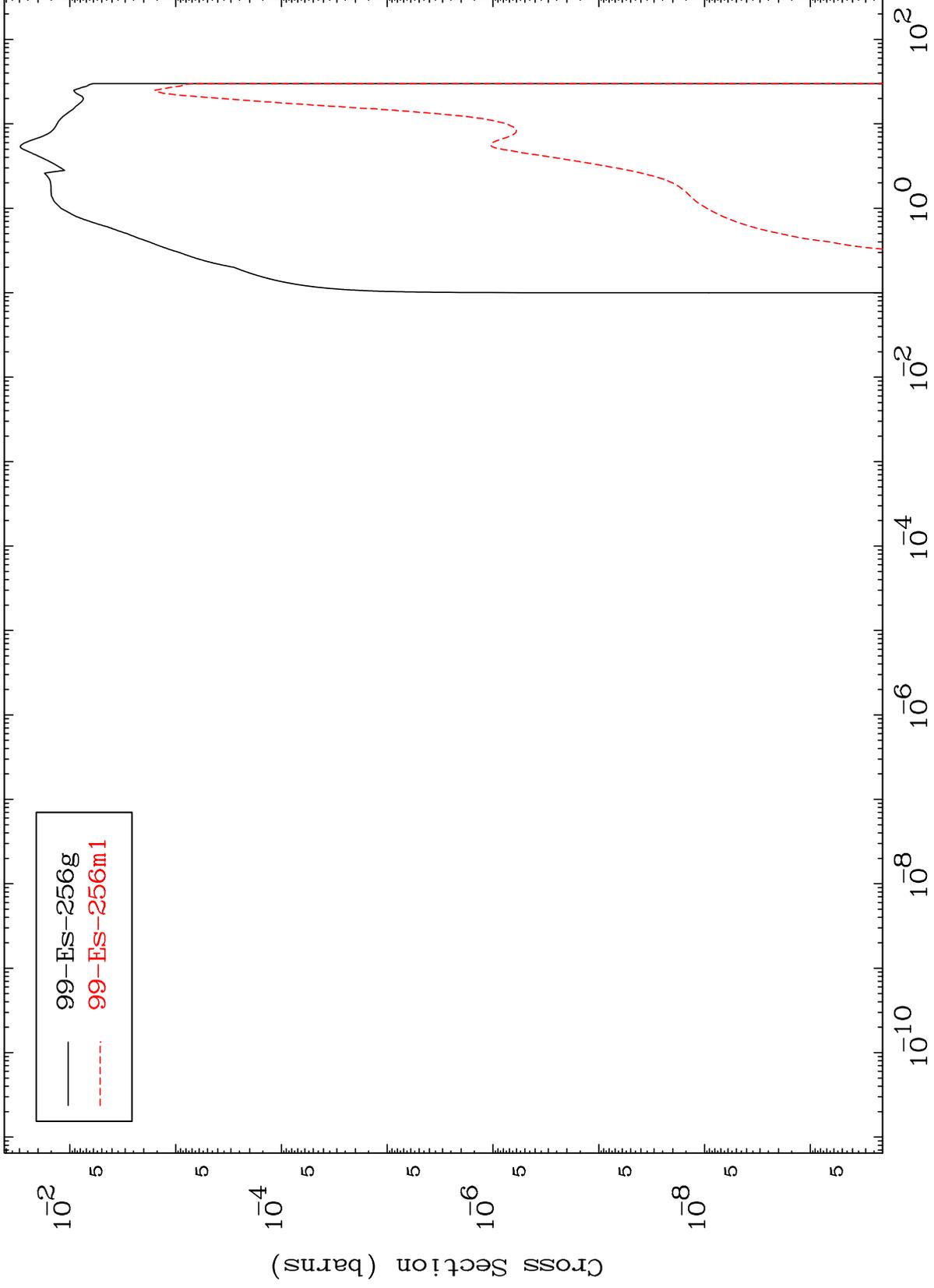


MAT 9961

(n,n') α

101-Md-260

Radionuclide Production Cross Section



26

Incident Energy (MeV)

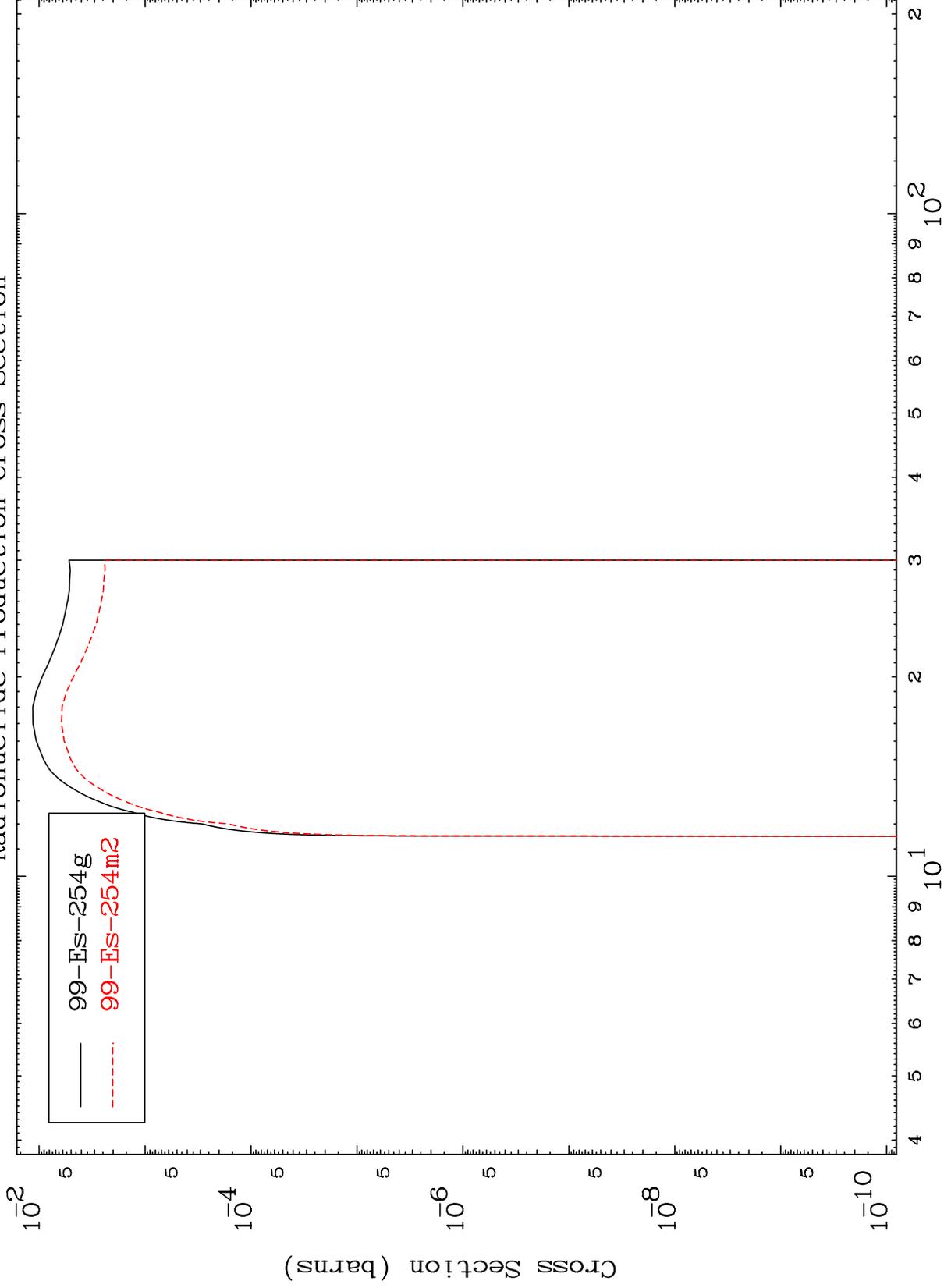
101-Md-260

MAT 9961

(n,3n) α

101-Md-260

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-260

27