

Program EVALPLOT  
(Version 2017-1)

by

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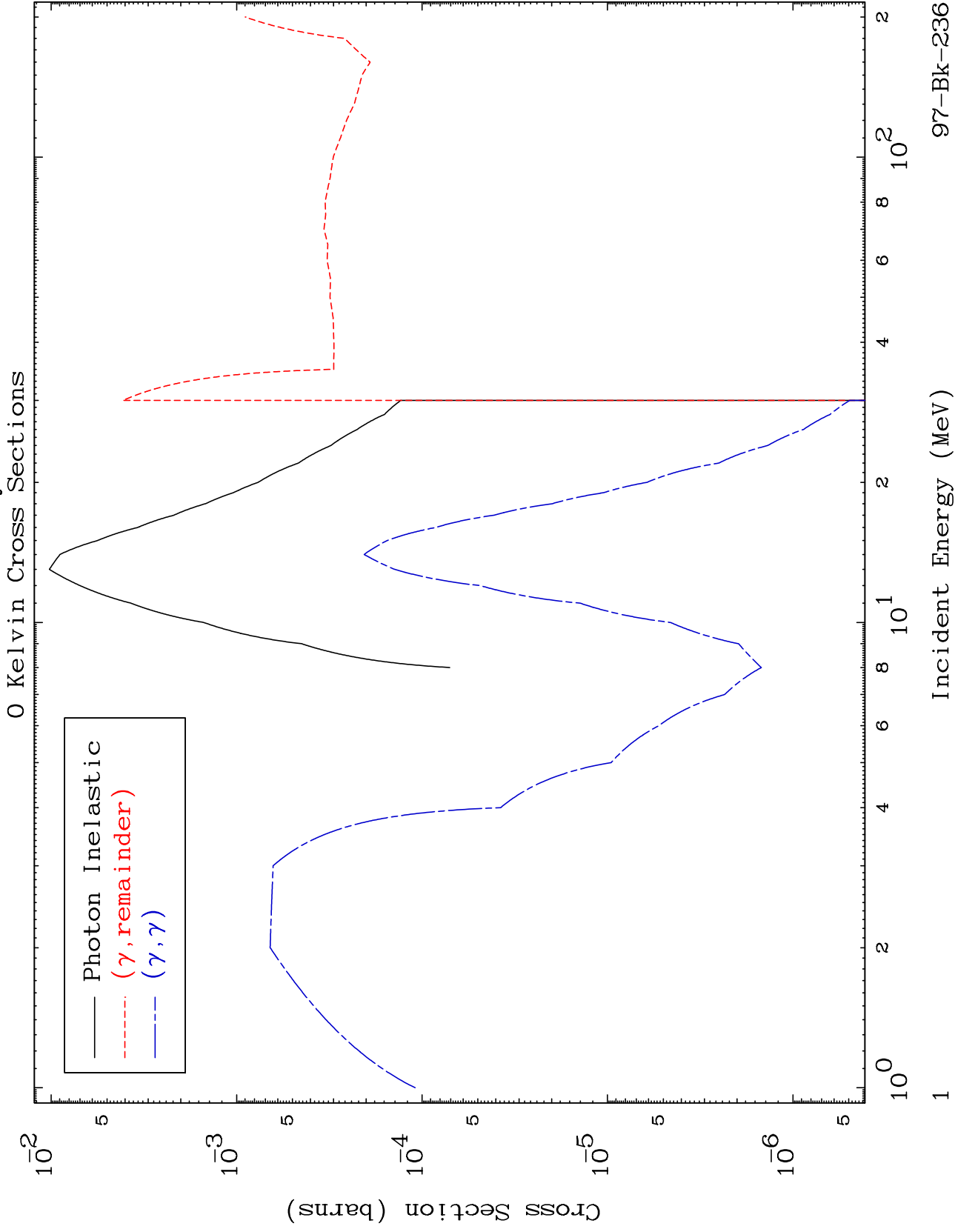
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Press Mouse Button to Start

MAT 9713

Photon Major

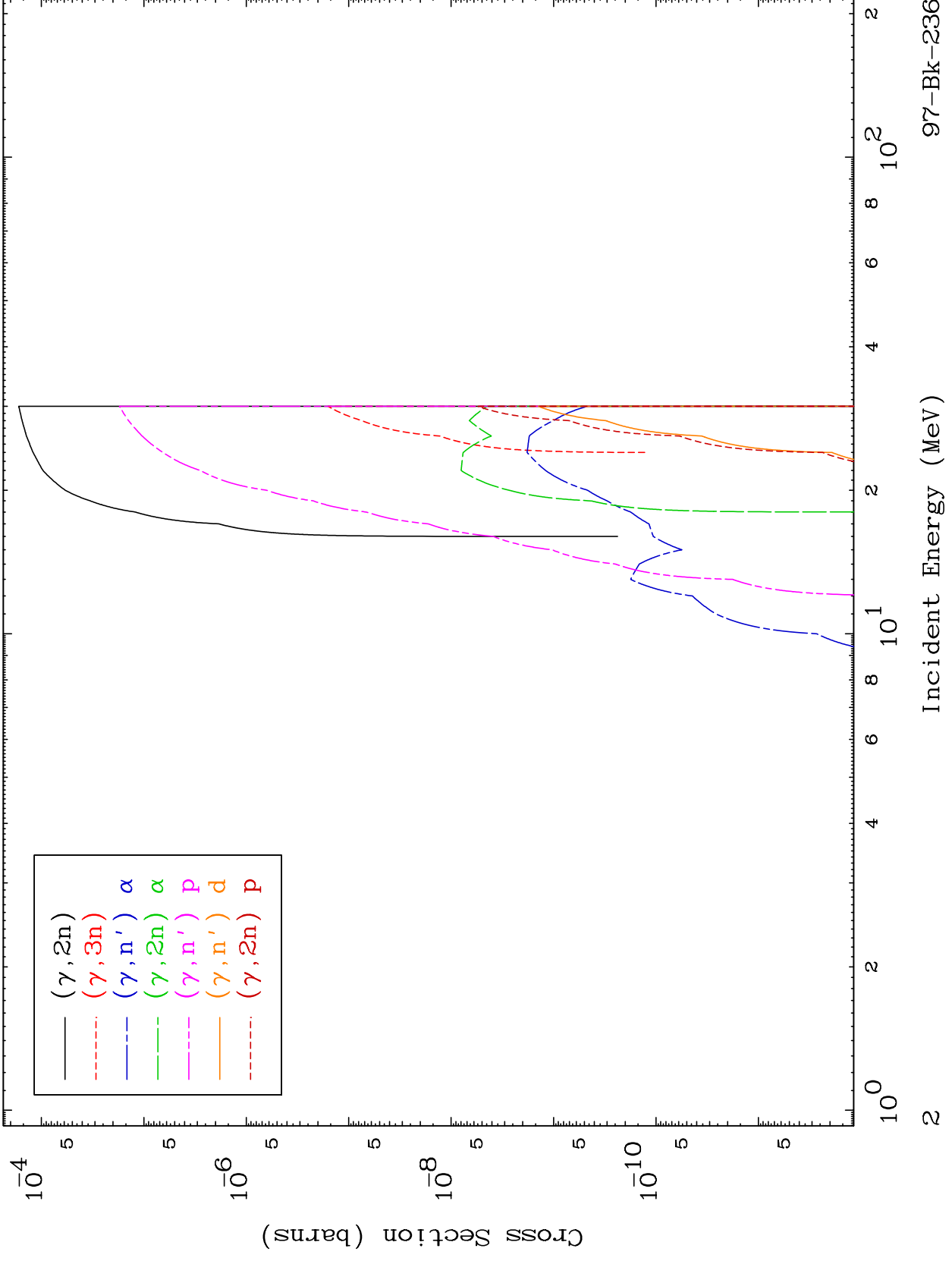
97-Bk-236



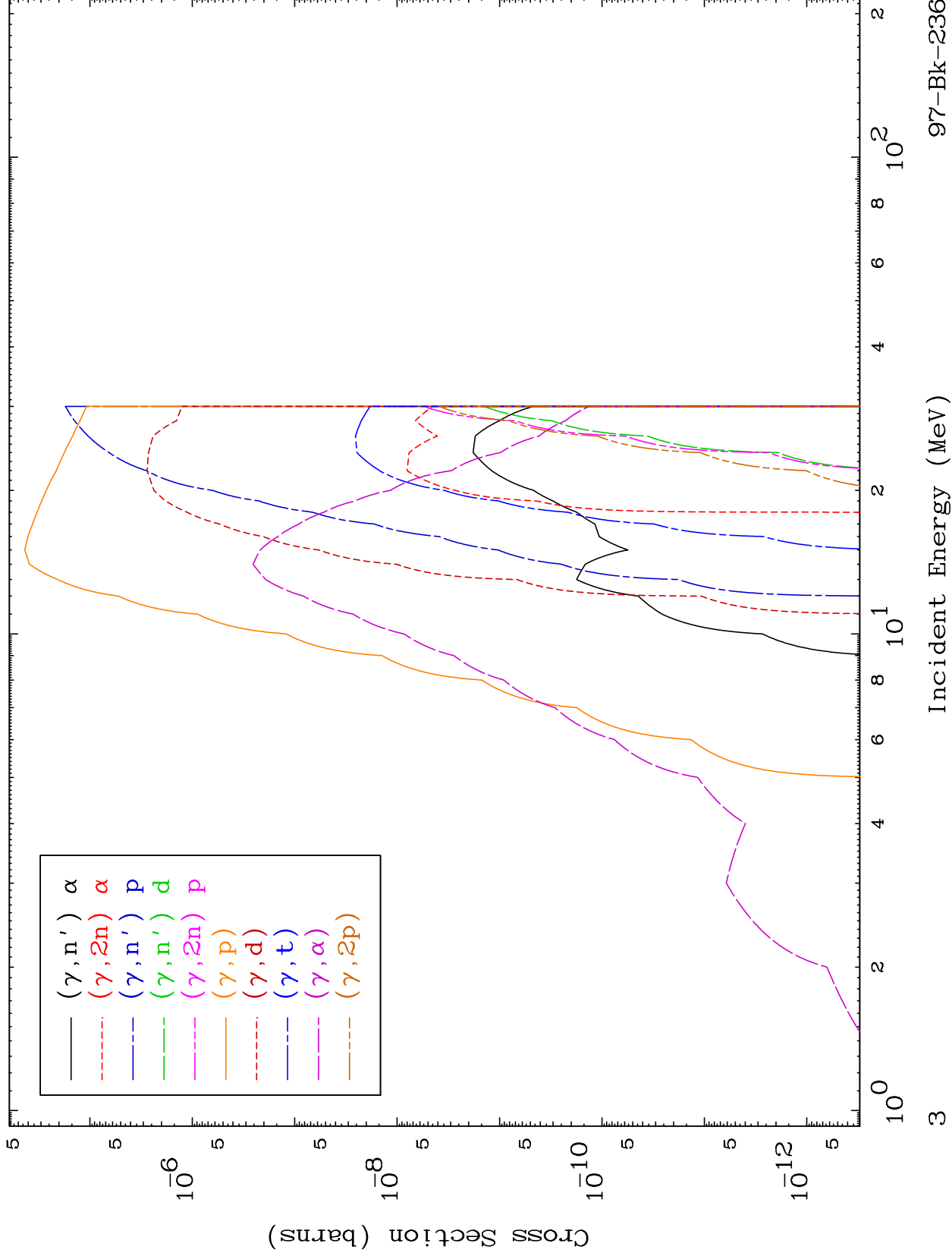
MAT 9713

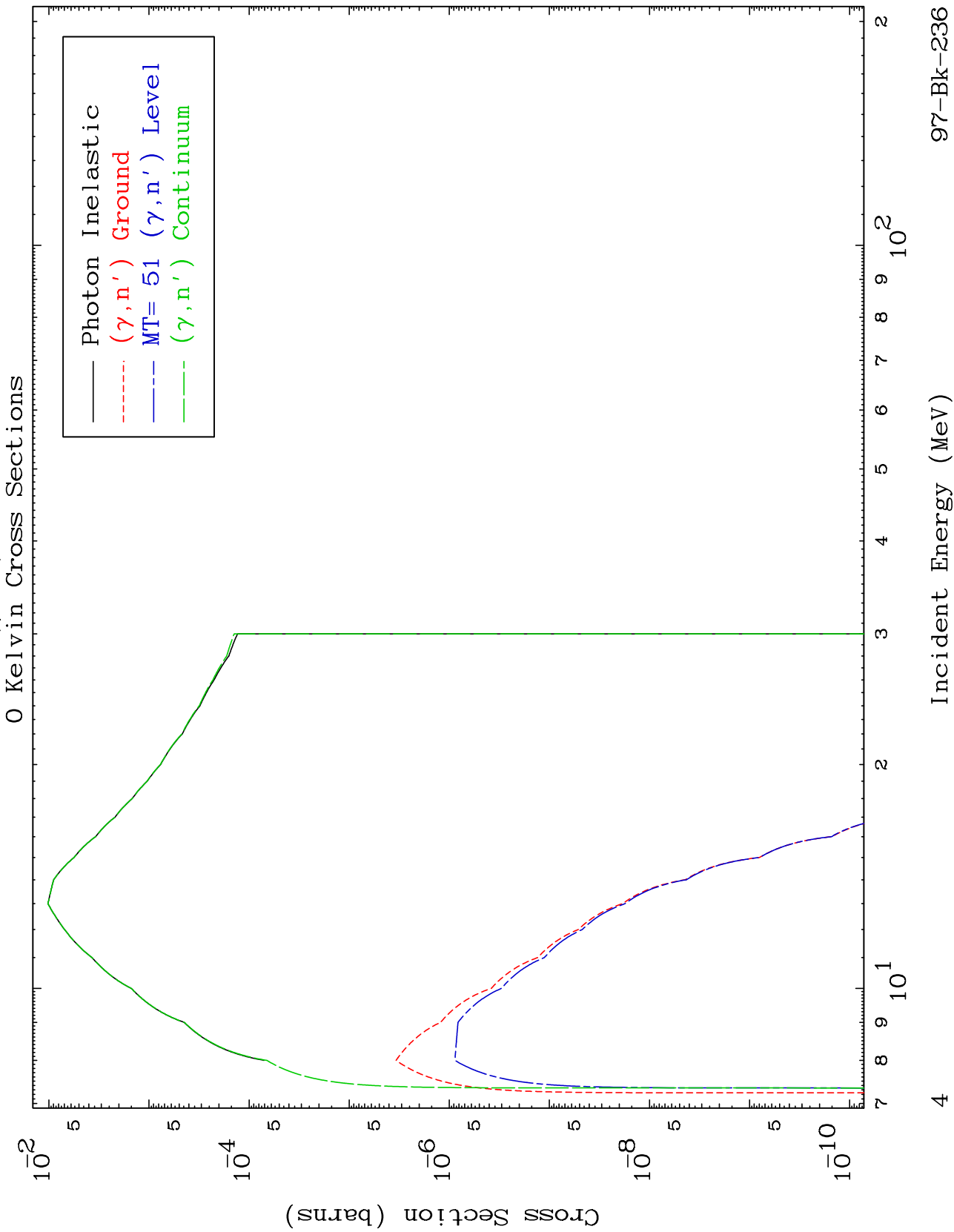
Photon Neutron Production  
0 Kelvin Cross Sections

97-Bk-236

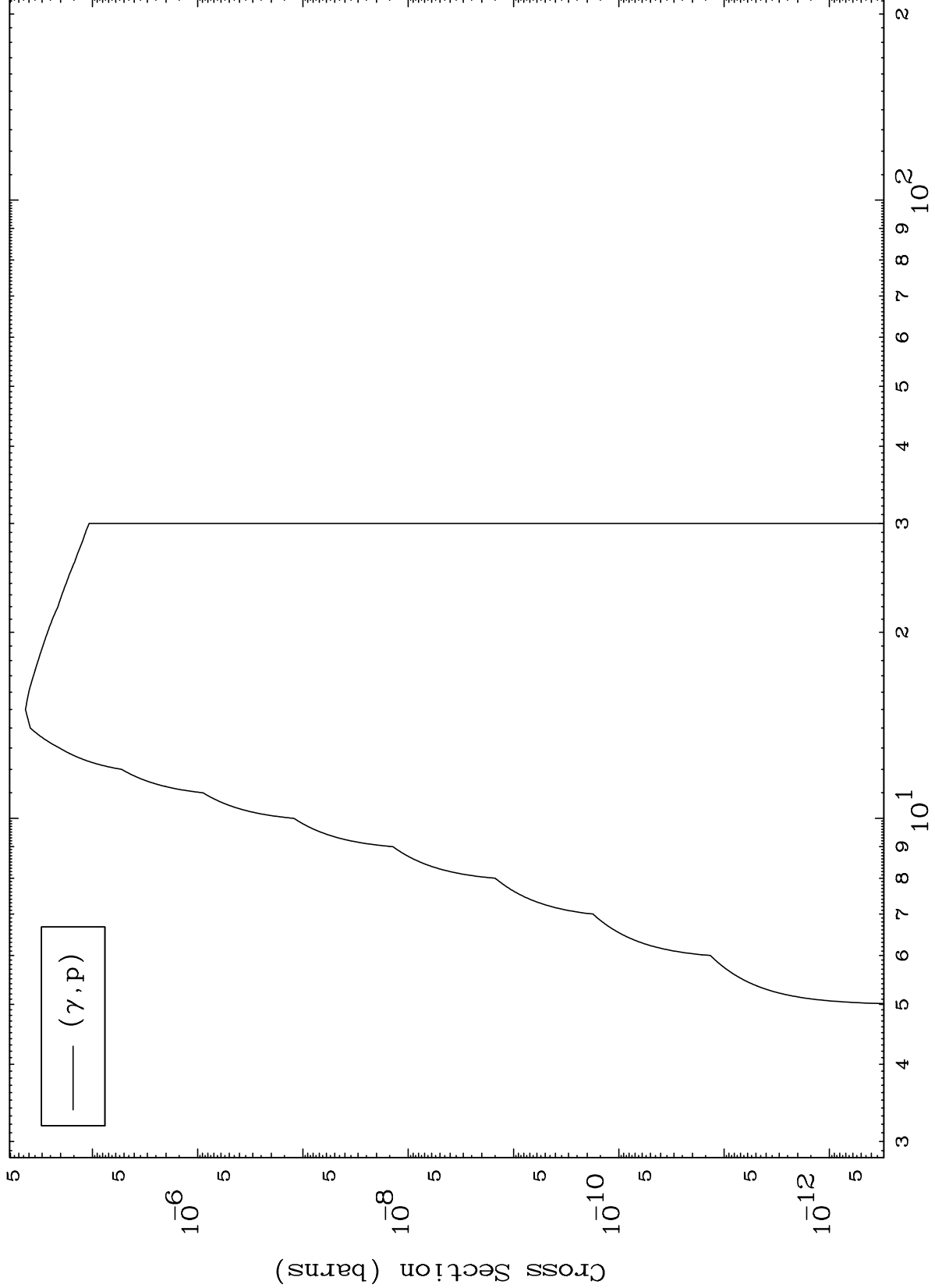


97-Bk-236





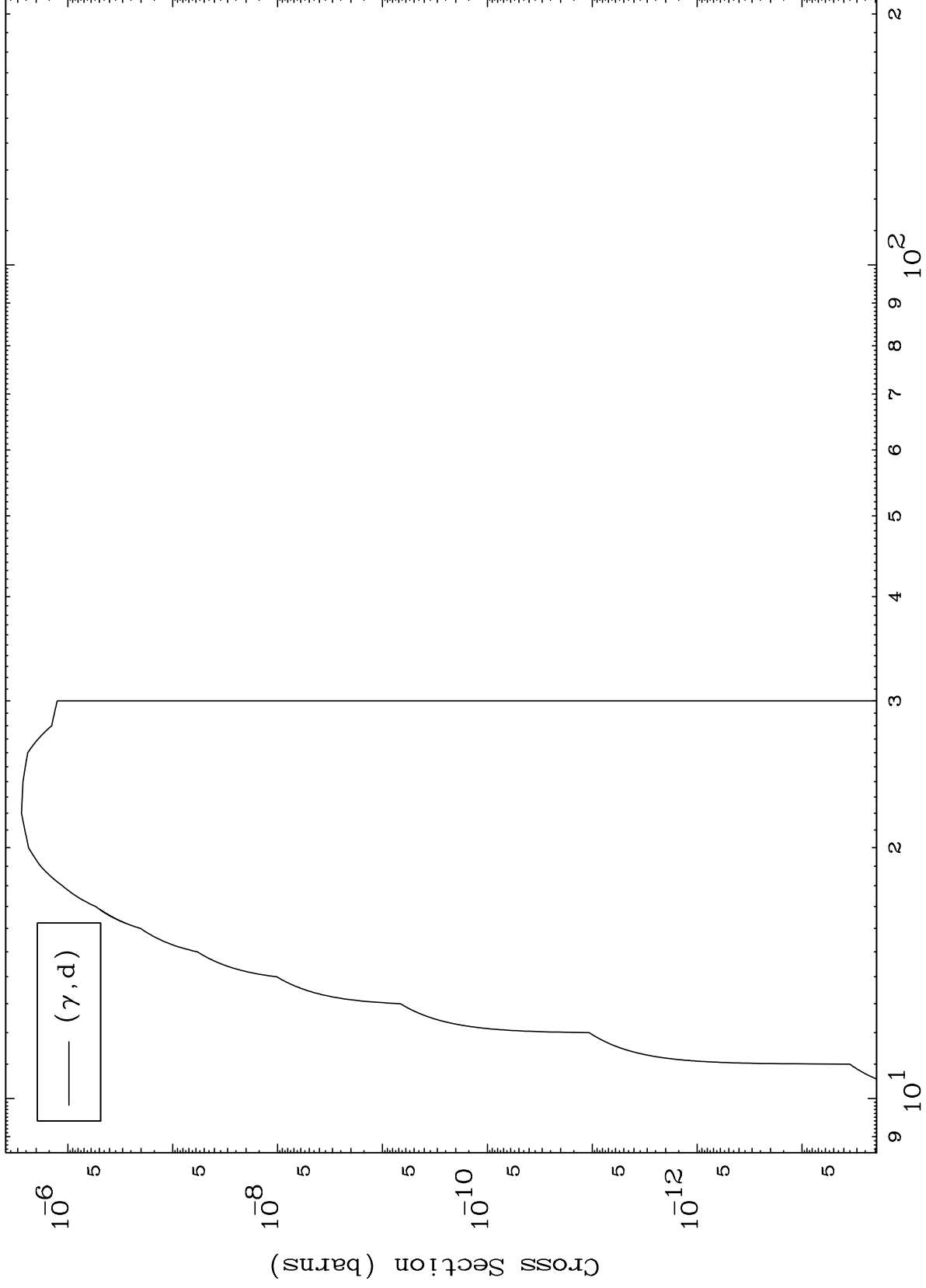
0 Kelvin Cross Sections



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( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

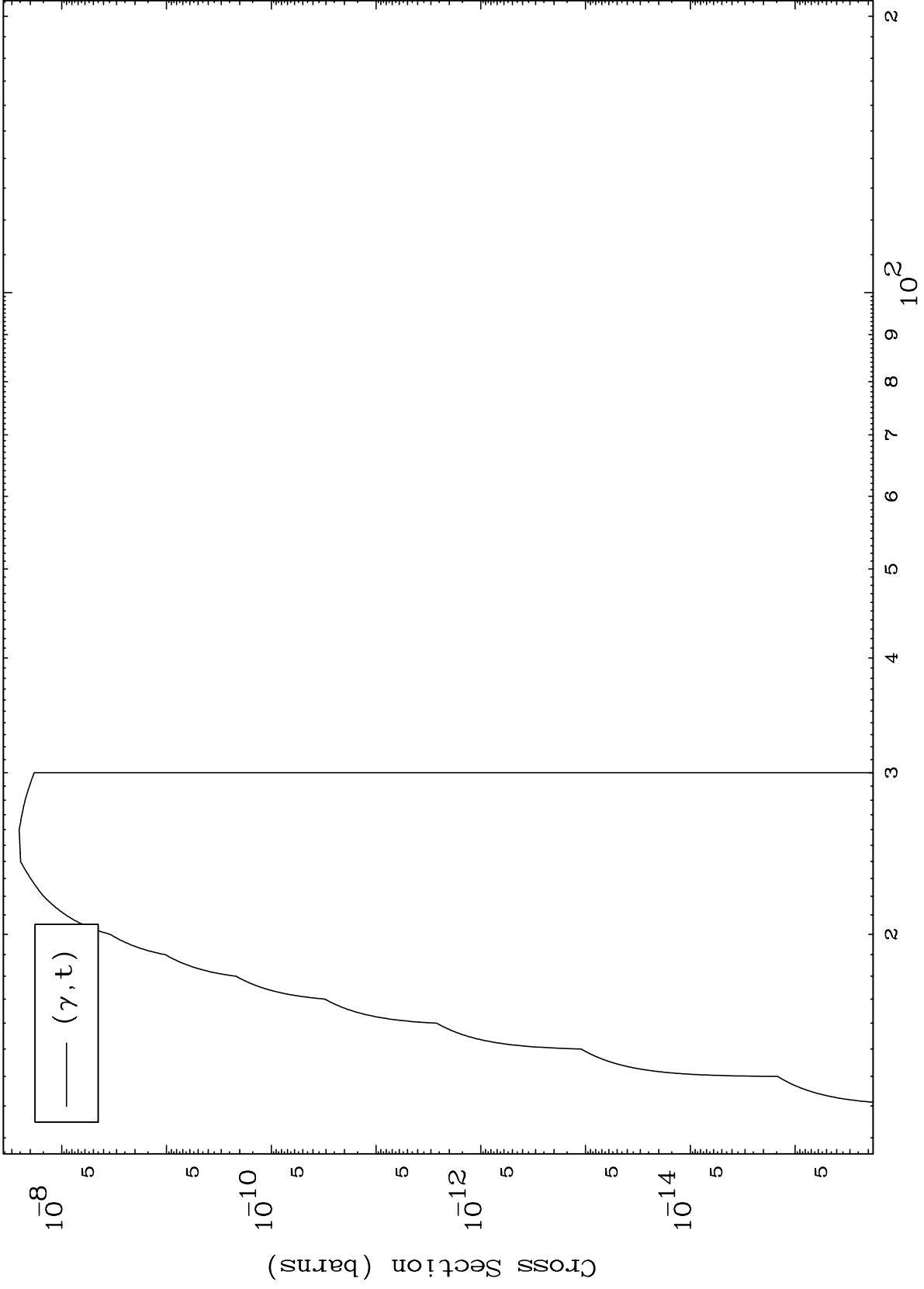
97-Bk-236



Incident Energy (MeV)

97-Bk-236

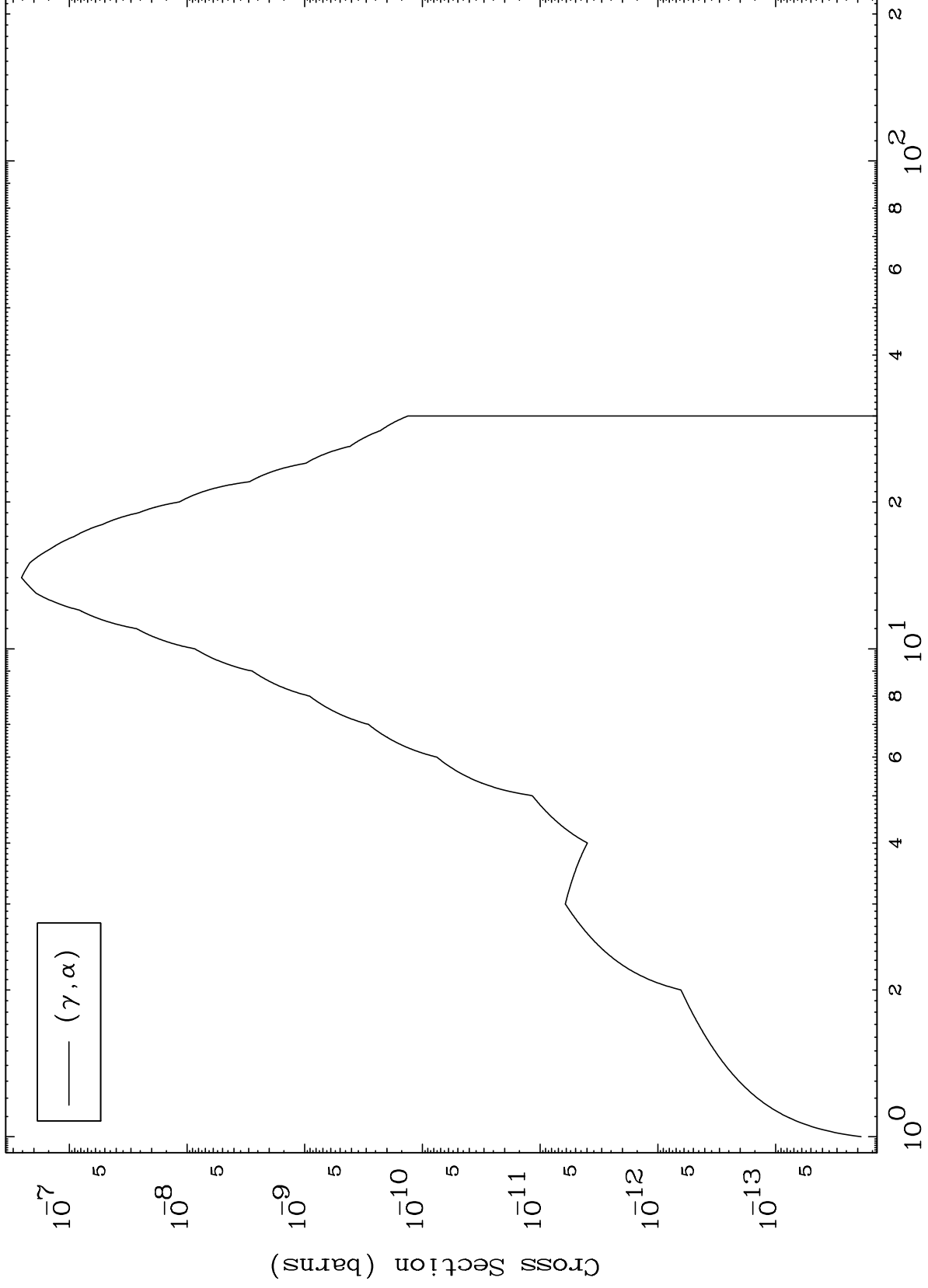
6



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( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

97-Bk-236



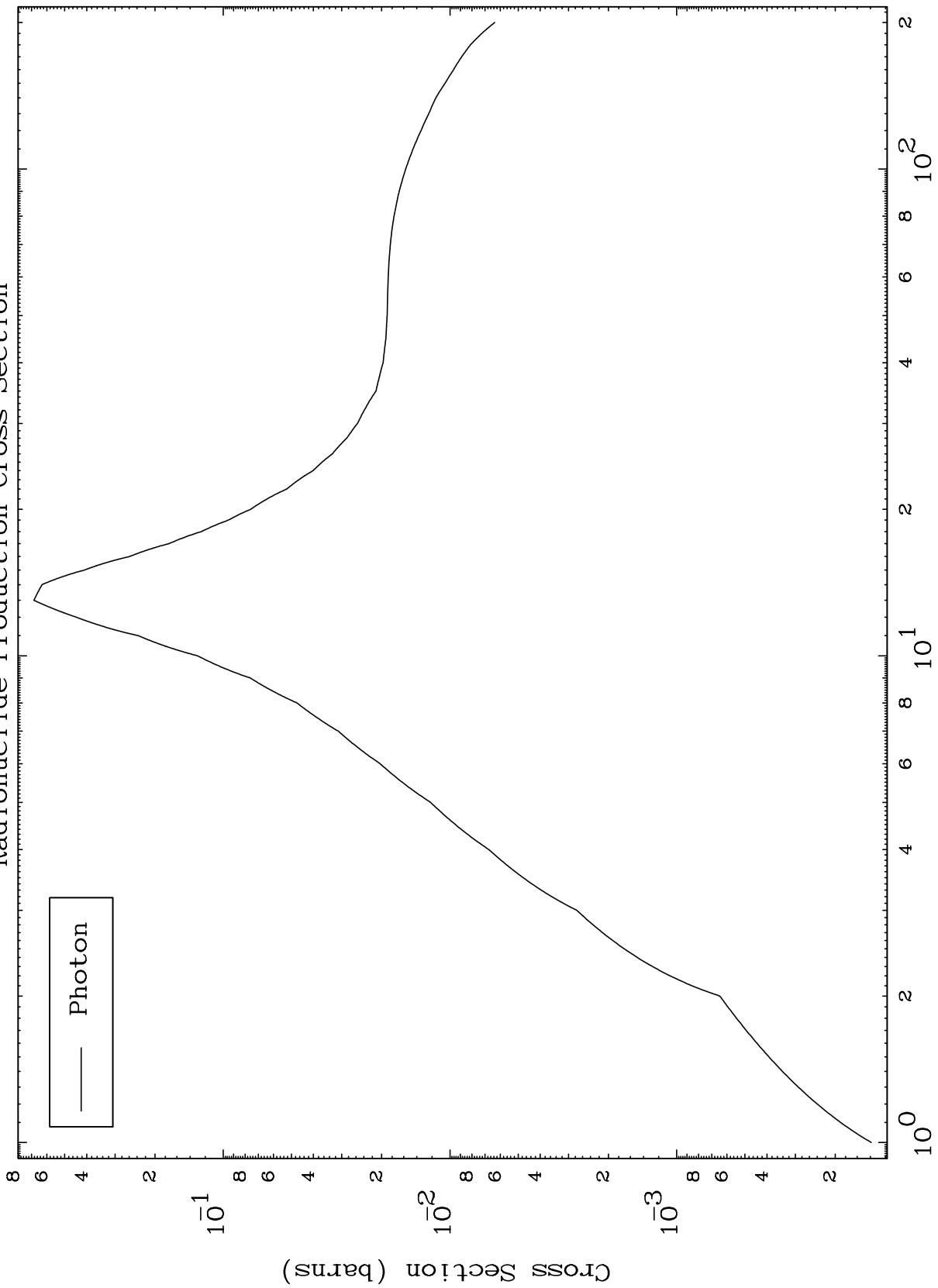
Incident Energy (MeV)

97-Bk-236

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Photon Fission  
Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-236