

Program EVALPLOT  
(Version 2017-1)

by

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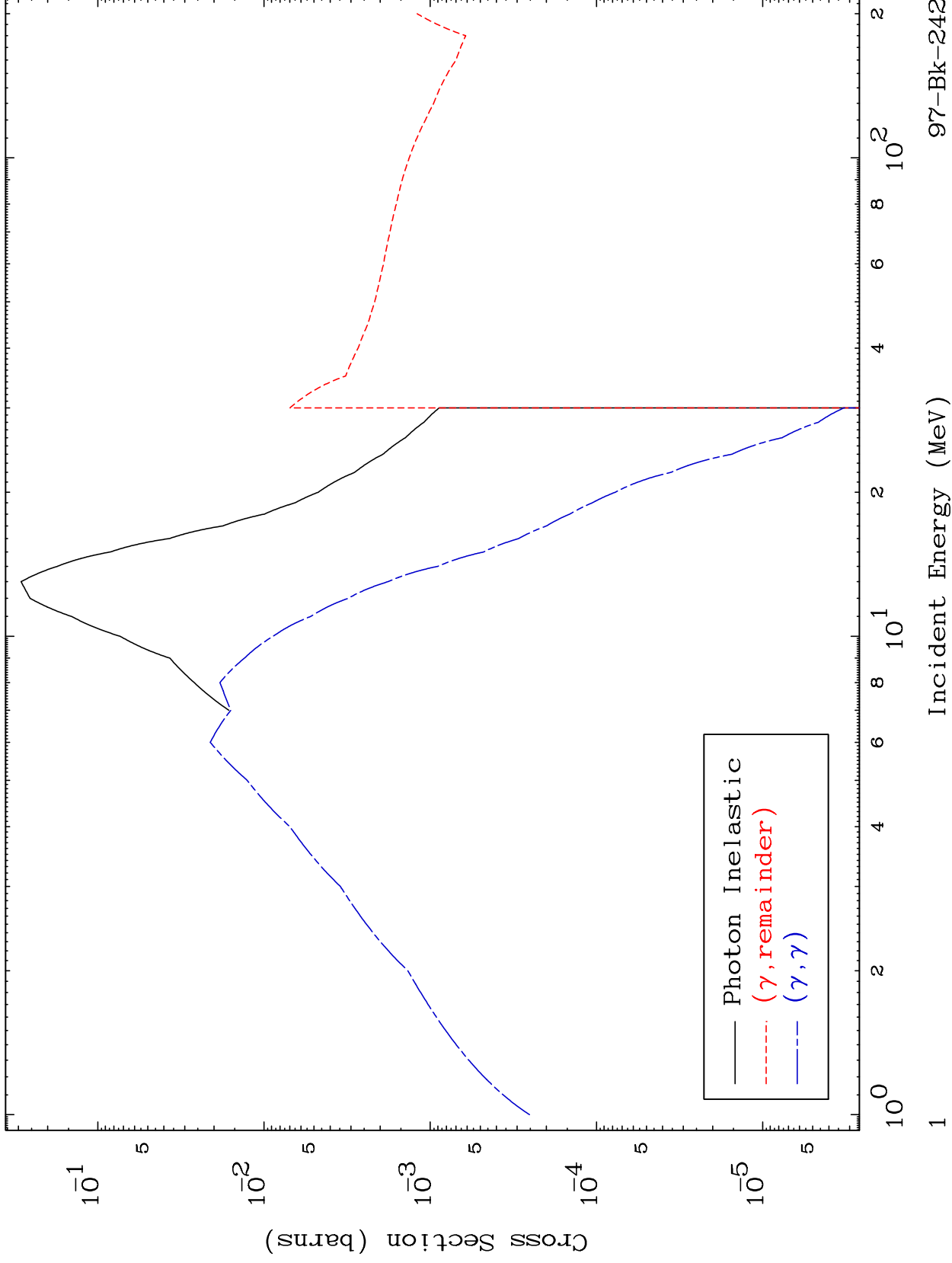
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9731

Photon Major  
0 Kelvin Cross Sections

97-Bk-242



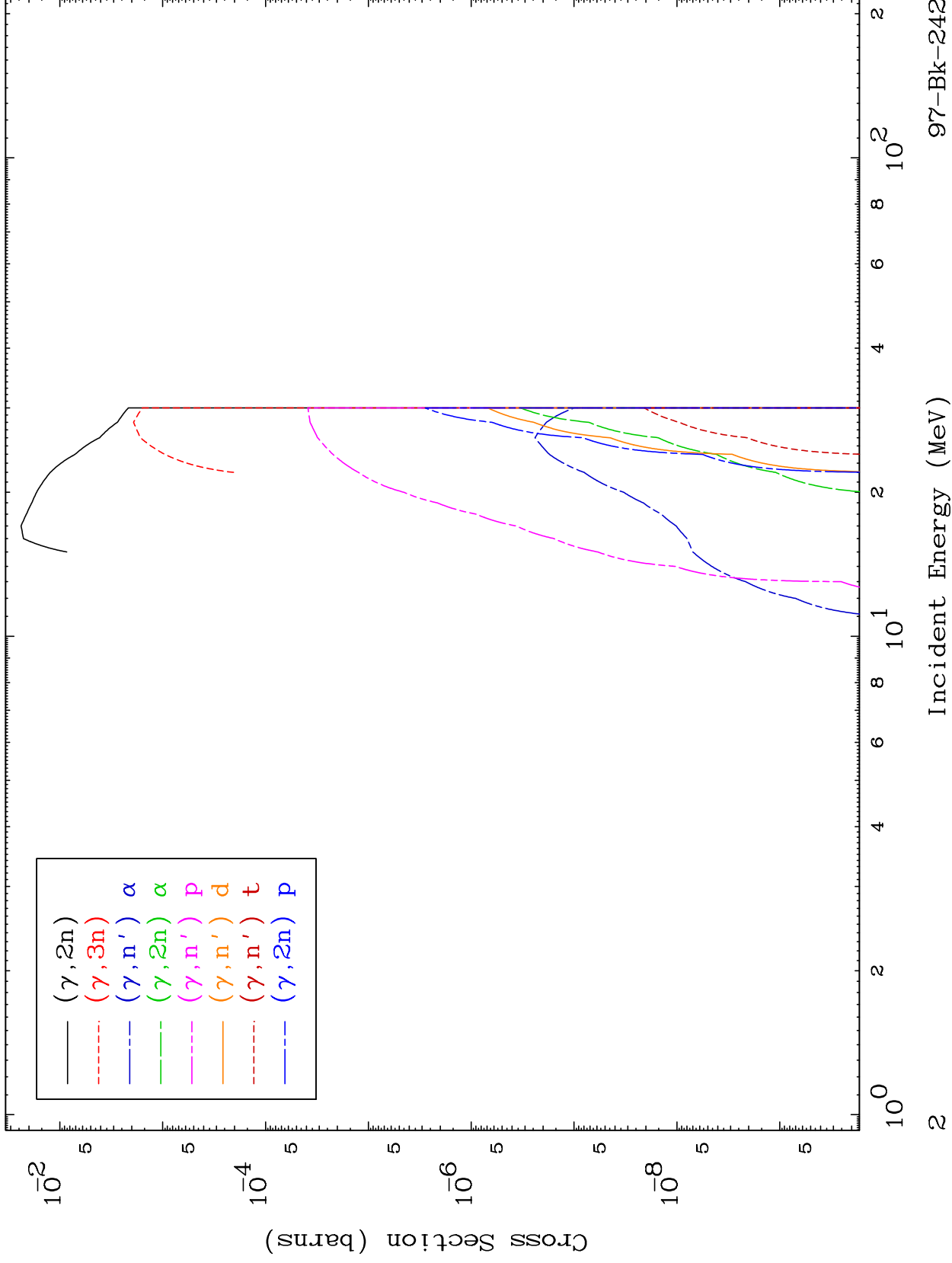
97-Bk-242

Incident Energy (MeV)

MAT 9731

Photon Neutron Production  
0 Kelvin Cross Sections

97-Bk-242

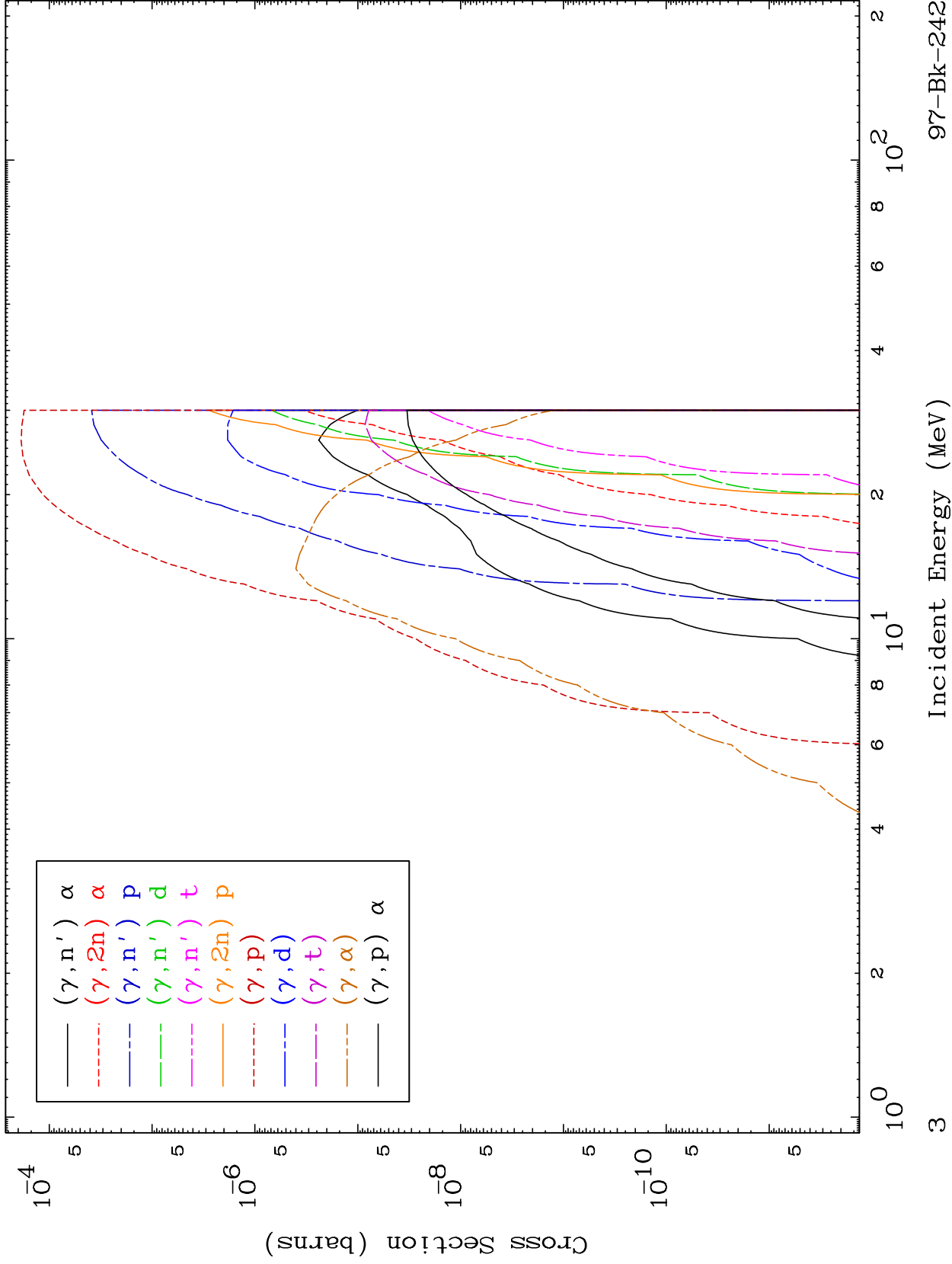


97-Bk-242

MAT 9731

Photon Charged Particle  
0 Kelvin Cross Sections

97-Bk-242

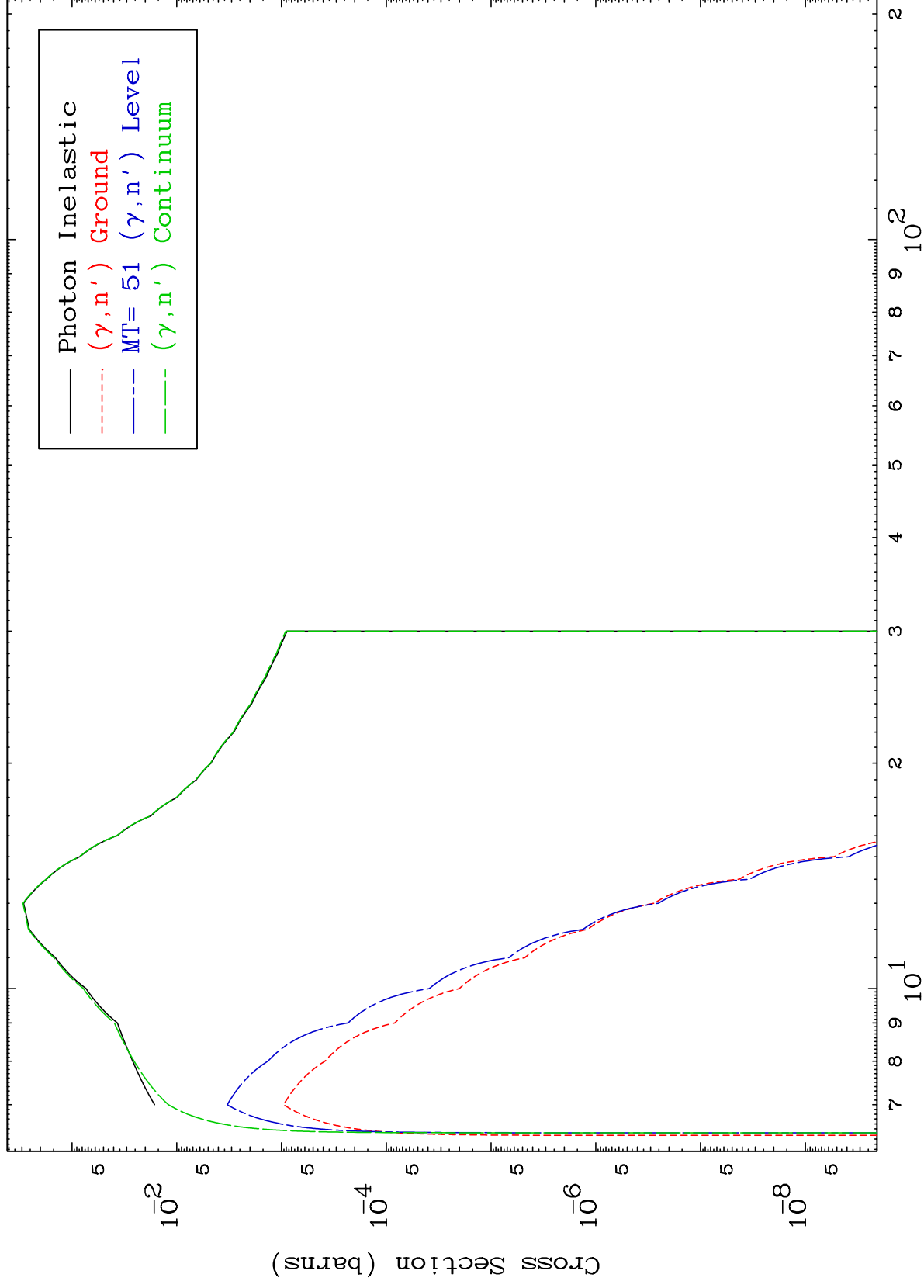


MAT 9731

( $\gamma, n'$ ) Level

97-Bk-242

0 Kelvin Cross Sections



4

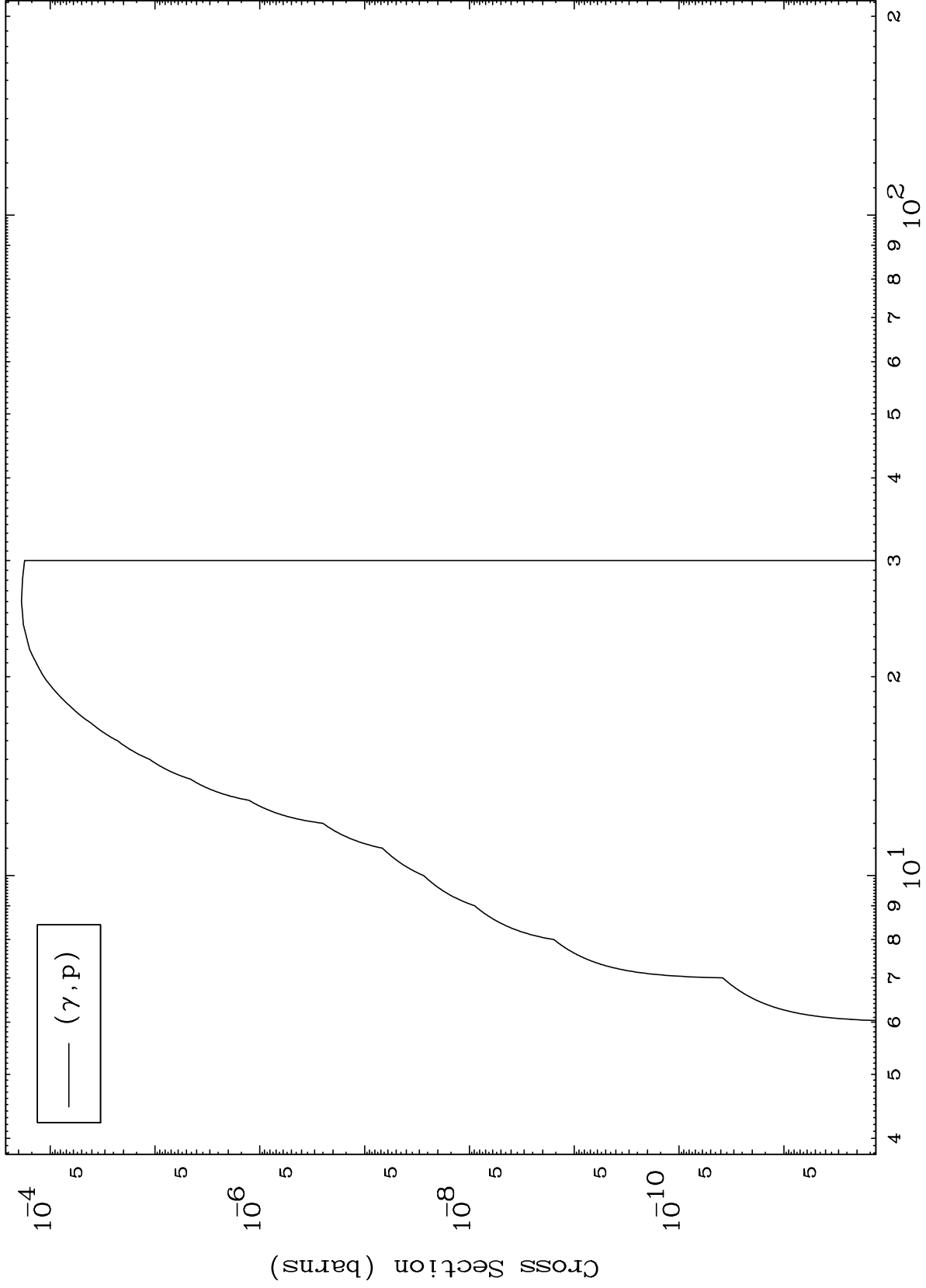
Incident Energy (MeV)

97-Bk-242

MAT 9731

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

97-Bk-242



5

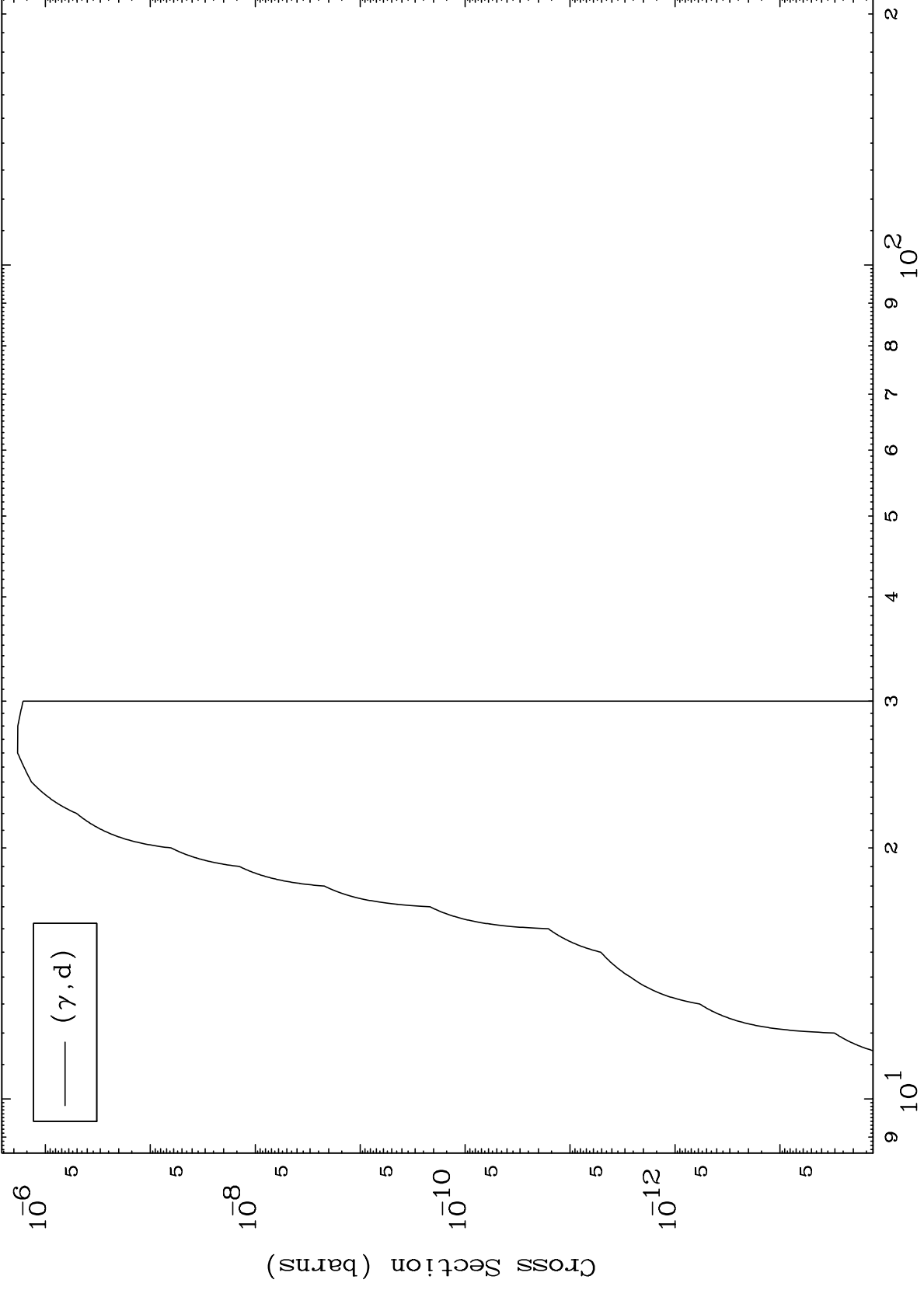
Incident Energy (MeV)

97-Bk-242

MAT 9731

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-242



97-Bk-242

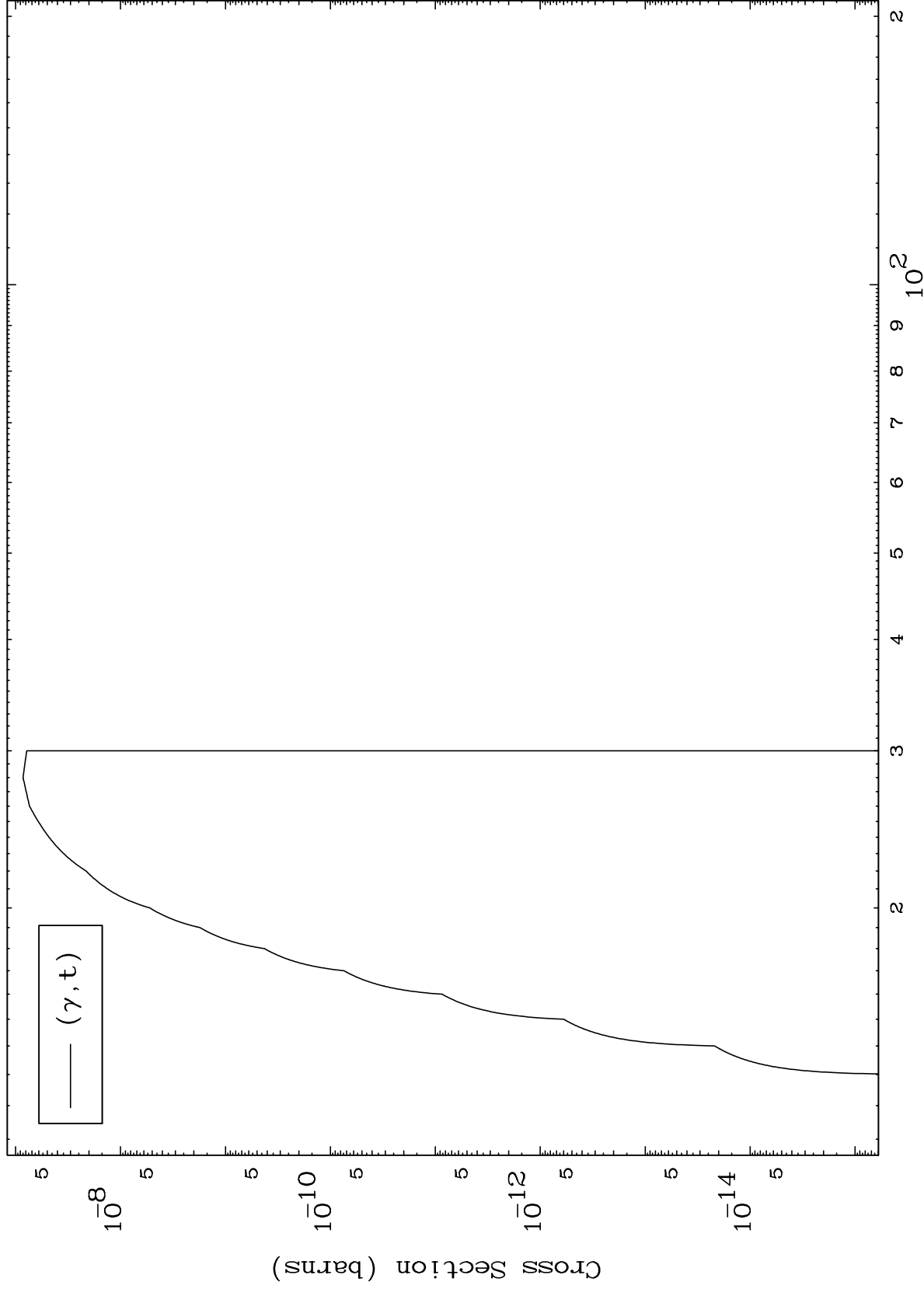
97-Bk-242

6

MAT 9731

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-242



7

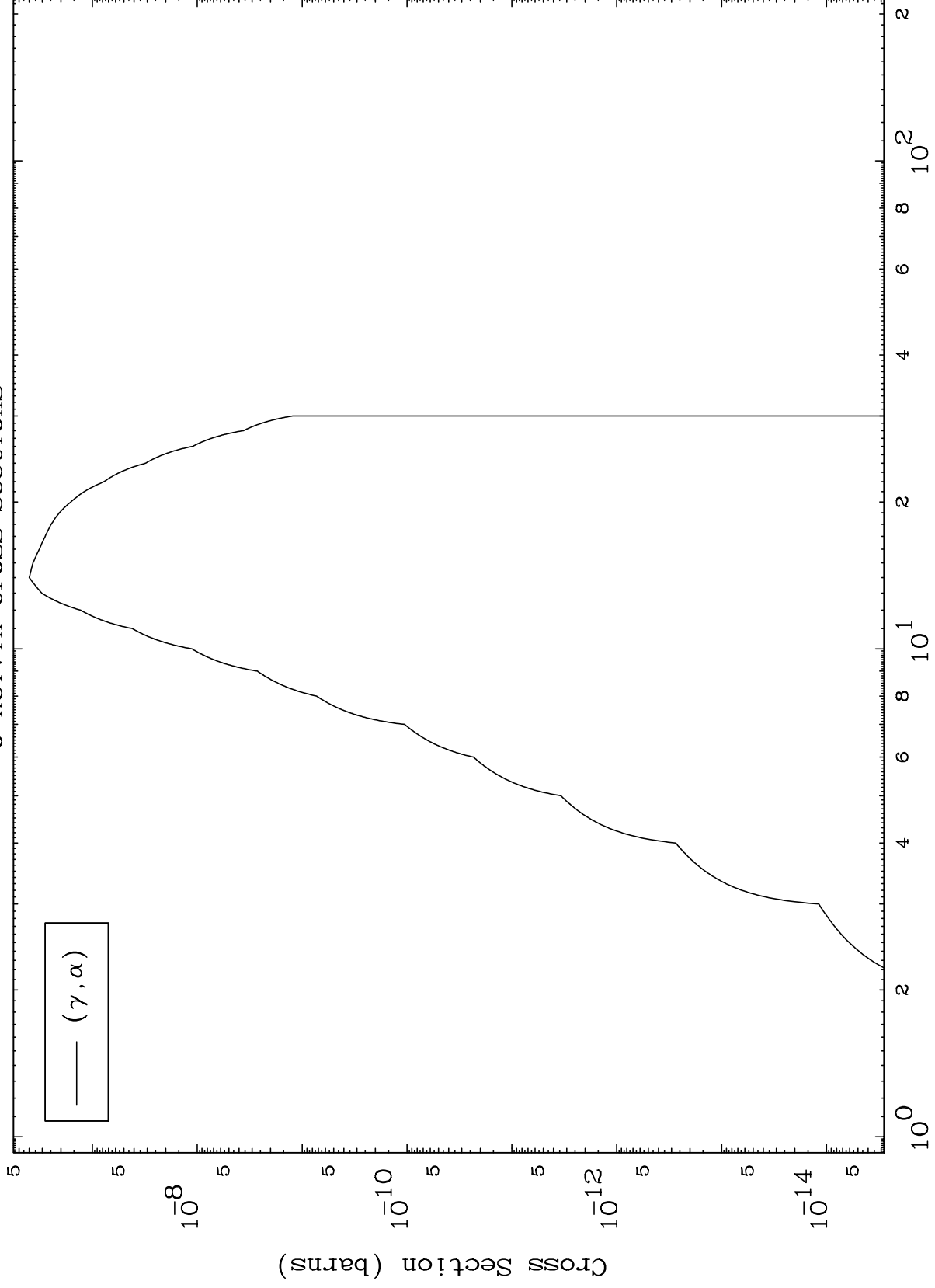
Incident Energy (MeV)

97-Bk-242

MAT 9731

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

97-Bk-242



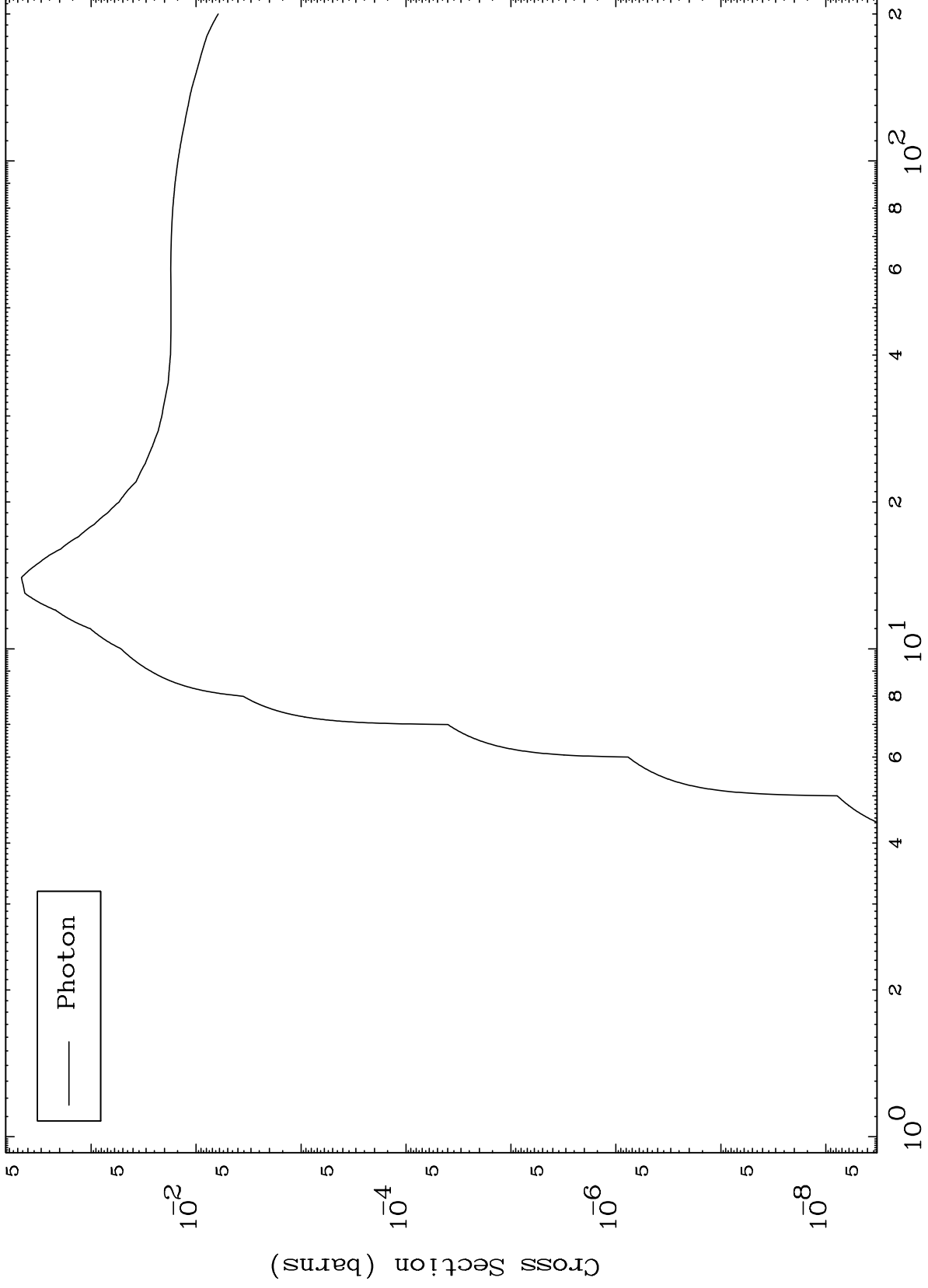
Incident Energy (MeV)

97-Bk-242

MAT 9731

Photon Fission  
Radionuclide Production Cross Section

97-Bk-242



Incident Energy (MeV)

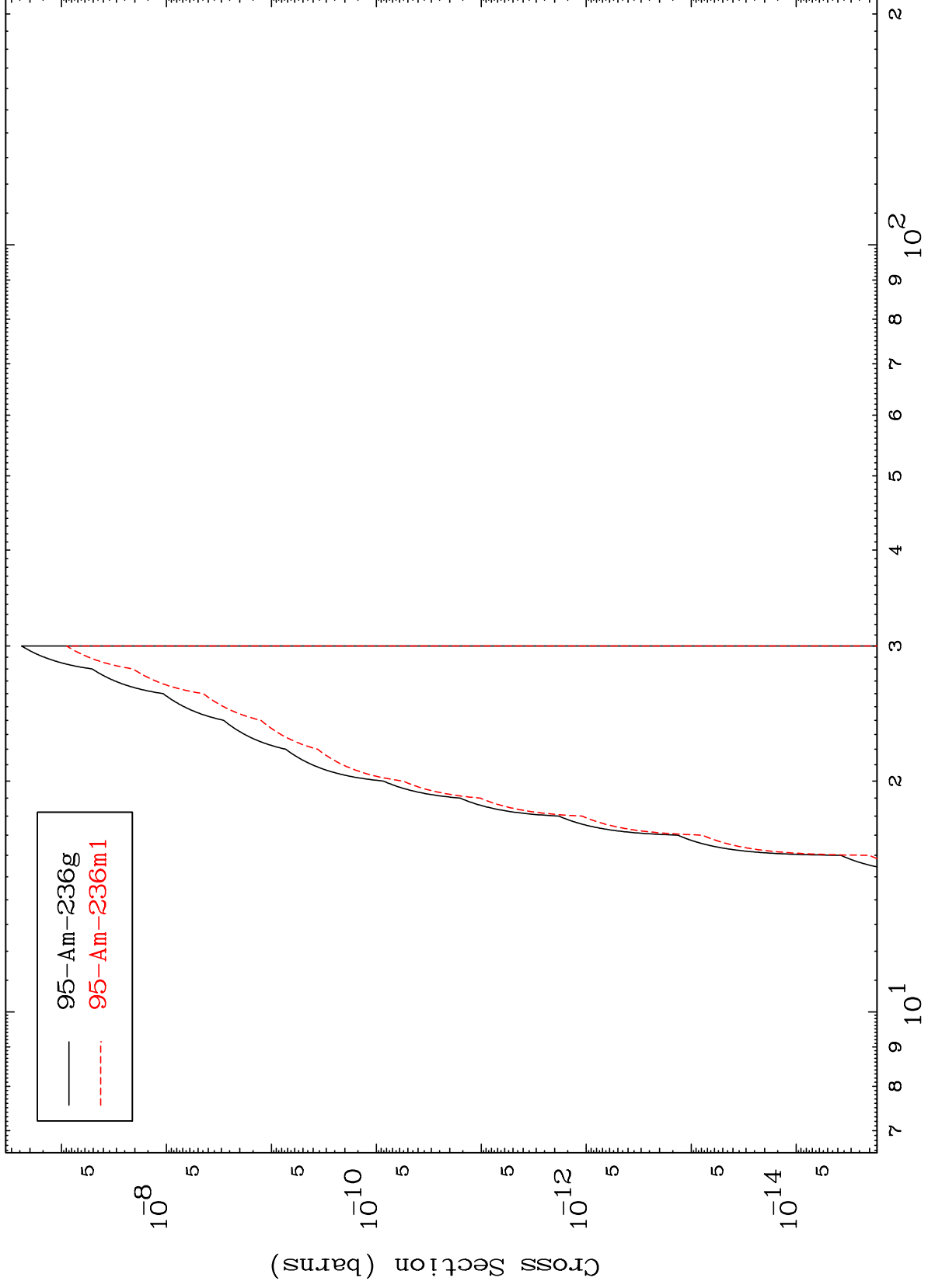
97-Bk-242

MAT 9731

( $\gamma, 2n$ )  $\alpha$

97-Bk-242

Radionuclide Production Cross Section



10

Incident Energy (MeV)

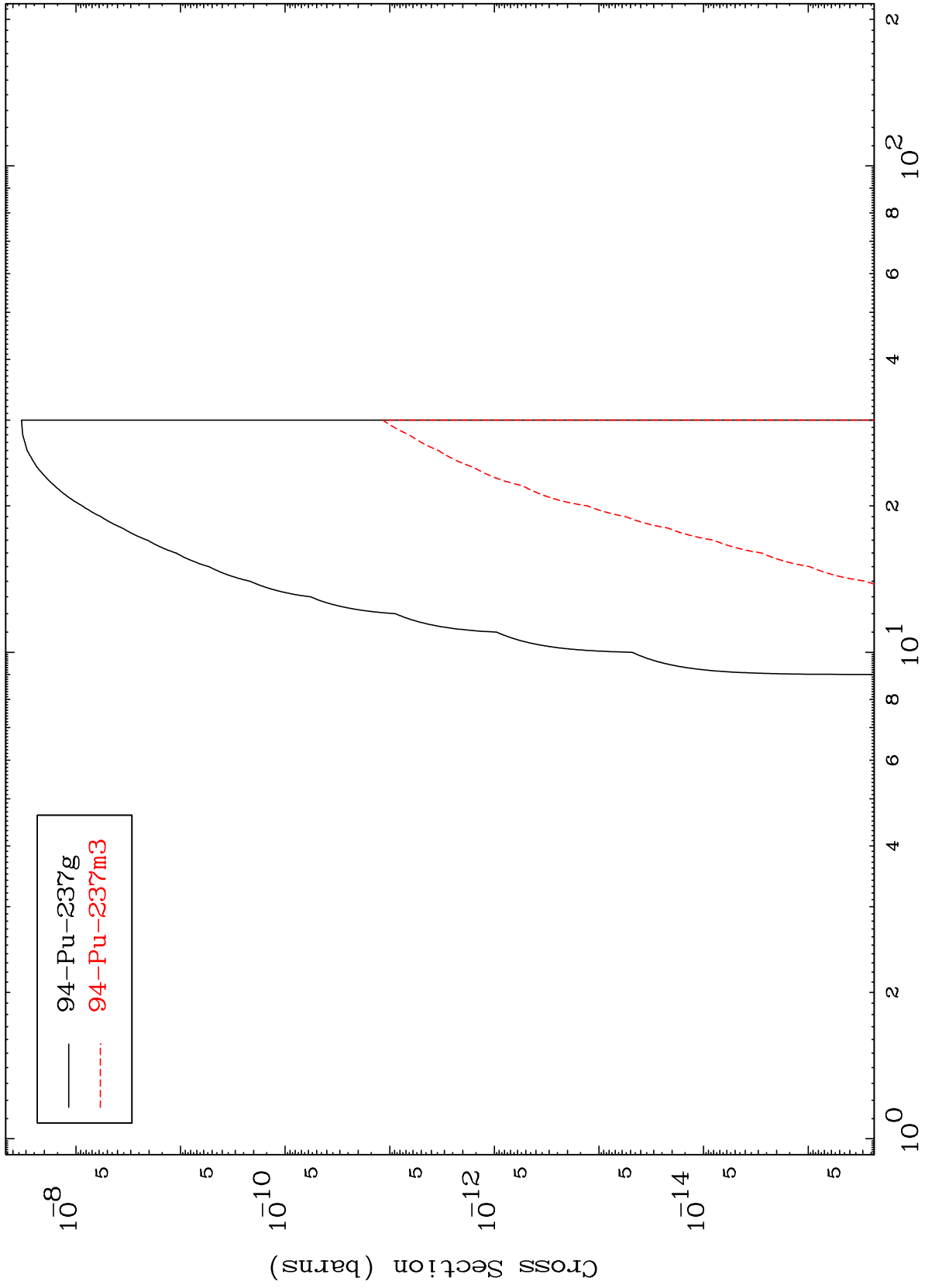
97-Bk-242

MAT 9731

( $\gamma, p$ )  $\alpha$

97-Bk-242

Radionuclide Production Cross Section



11

Incident Energy (MeV)

97-Bk-242