

Program EVALPLOT
(Version 2017-1)

by

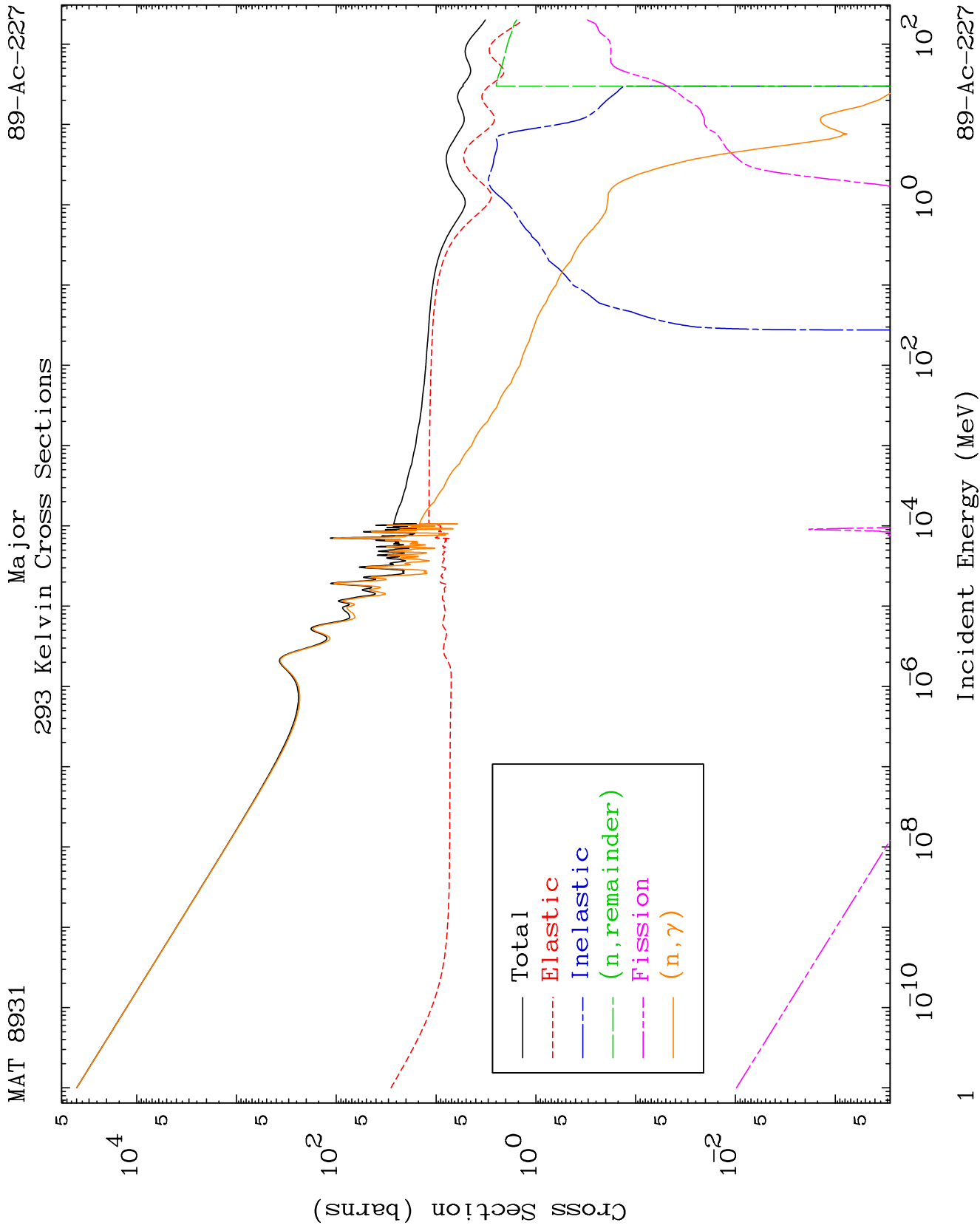
Dermott E. Cullen
(Present Contact Information)

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Tele: 925-443-1911

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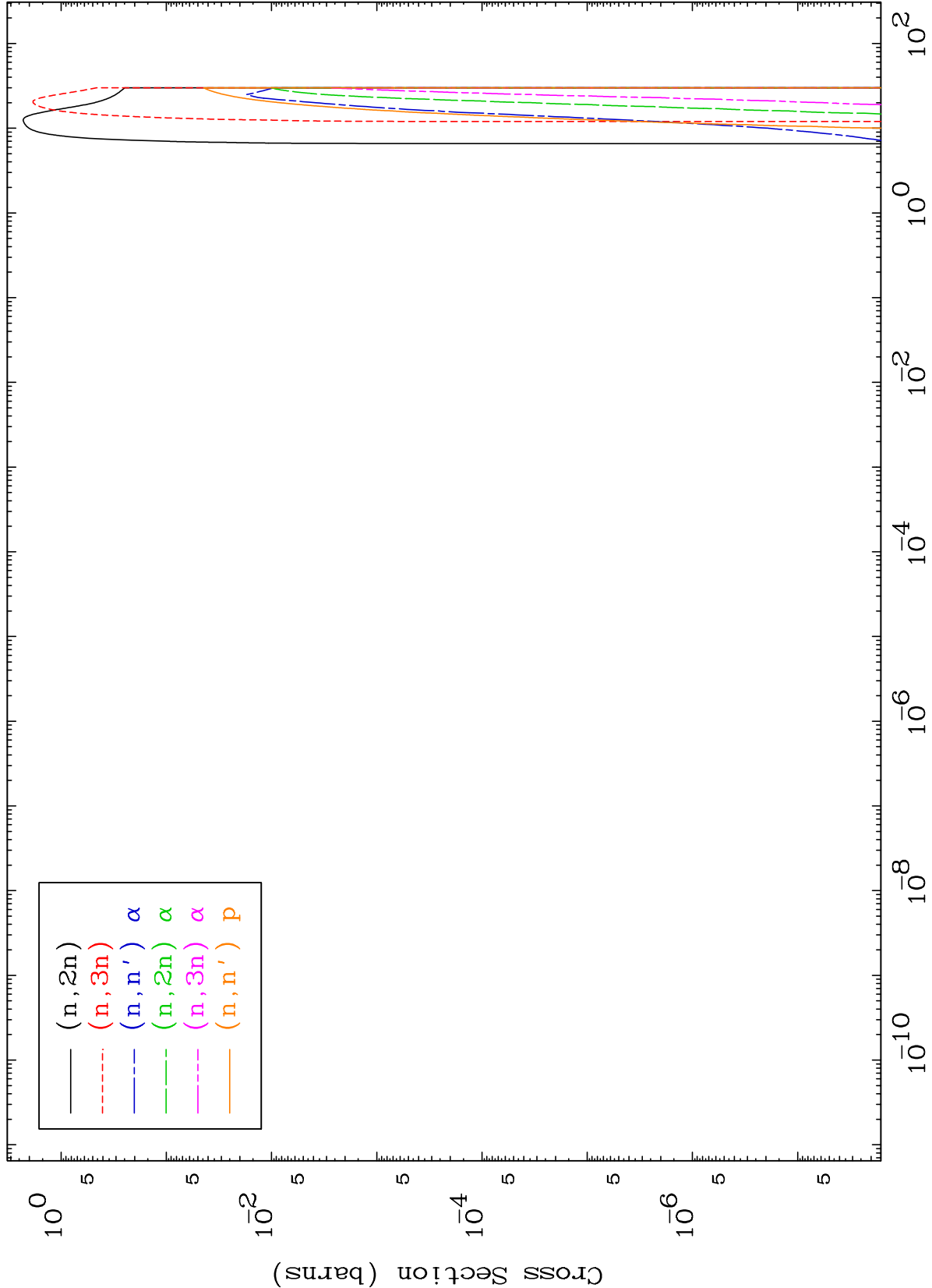
Press Mouse Button to Start



MAT 8931

Neutron Production
293 Kelvin Cross Sections

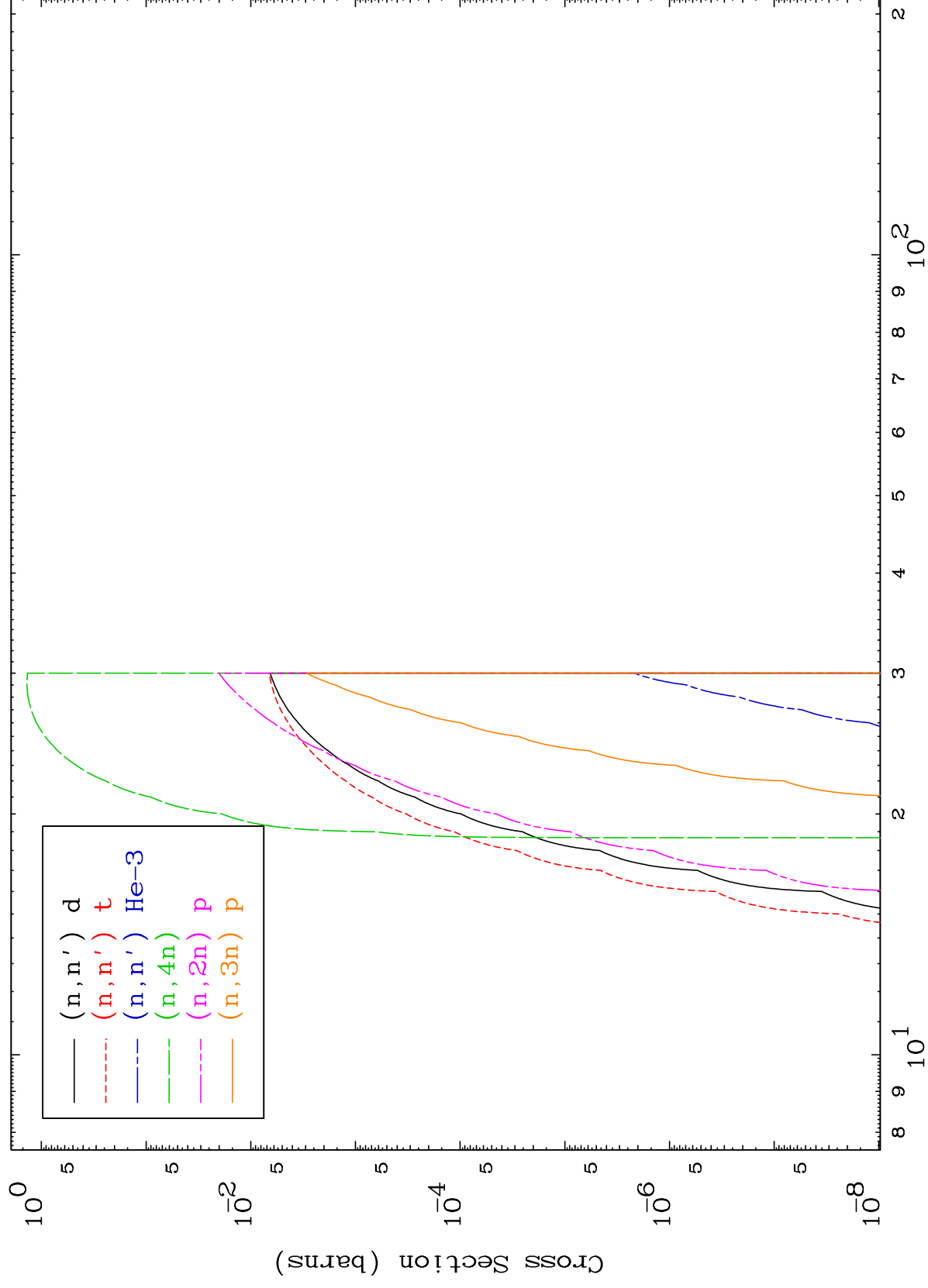
89-Ac-227



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Neutron Production 293 Kelvin Cross Sections

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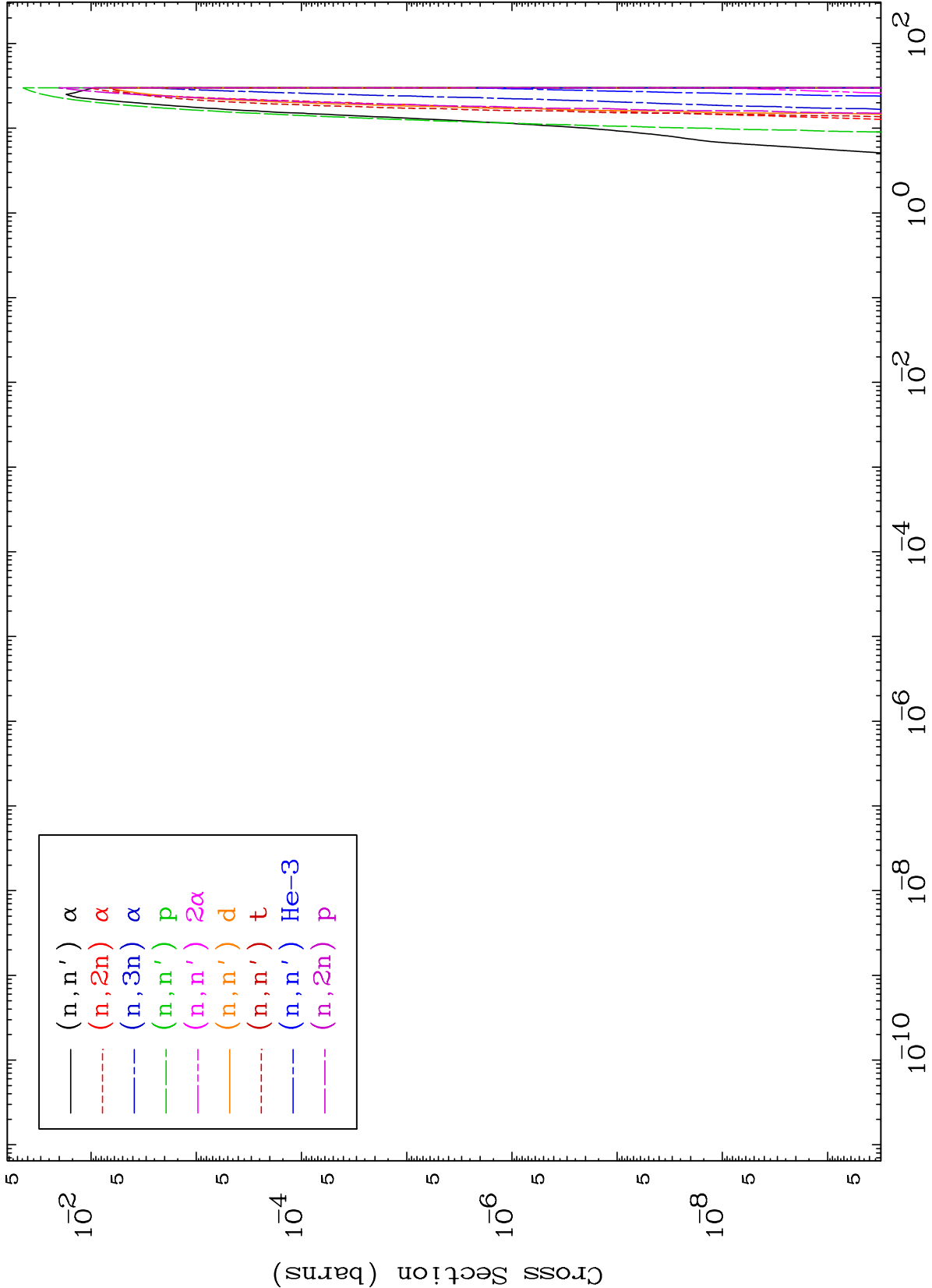


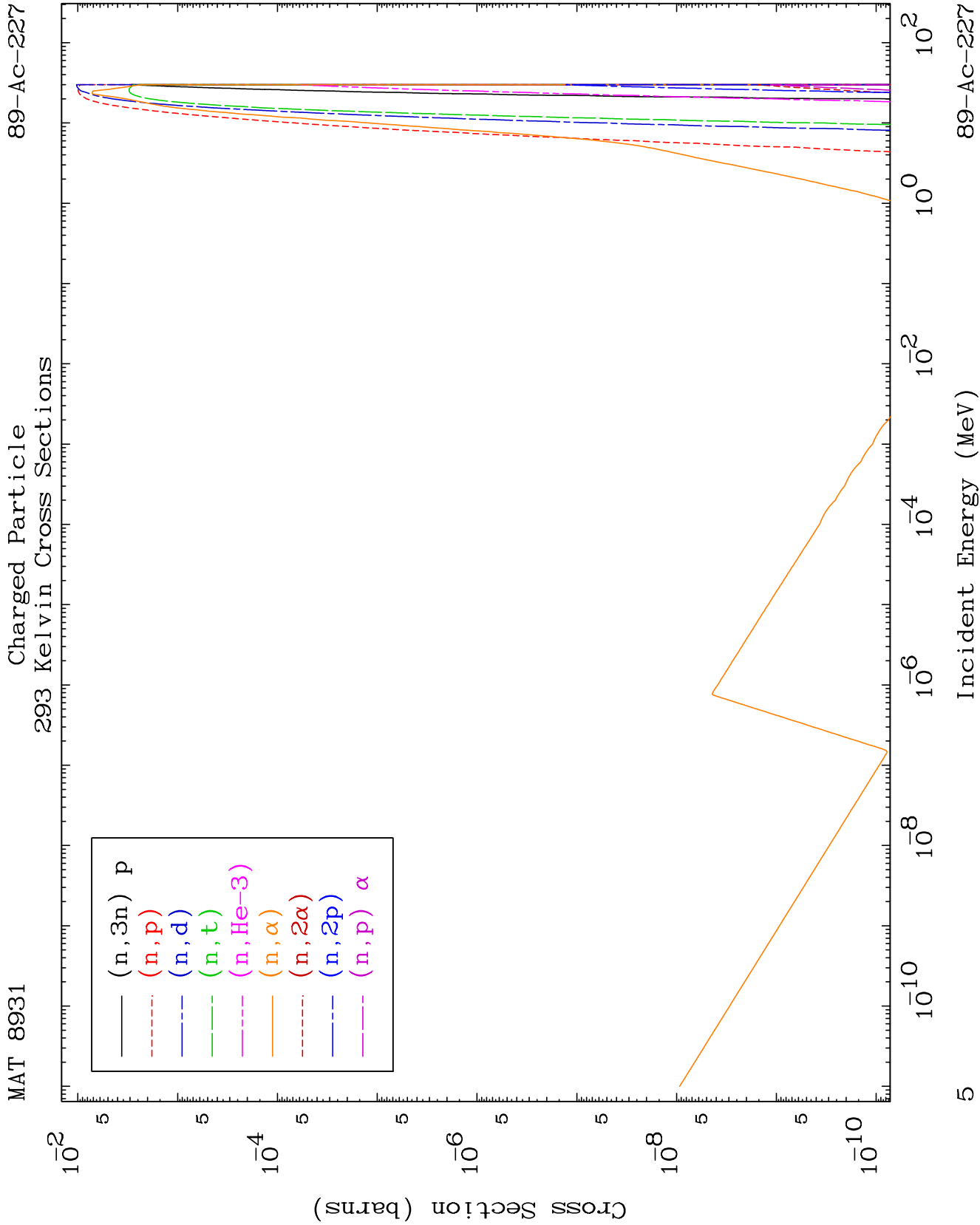
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Charged Particle
293 Kelvin Cross Sections

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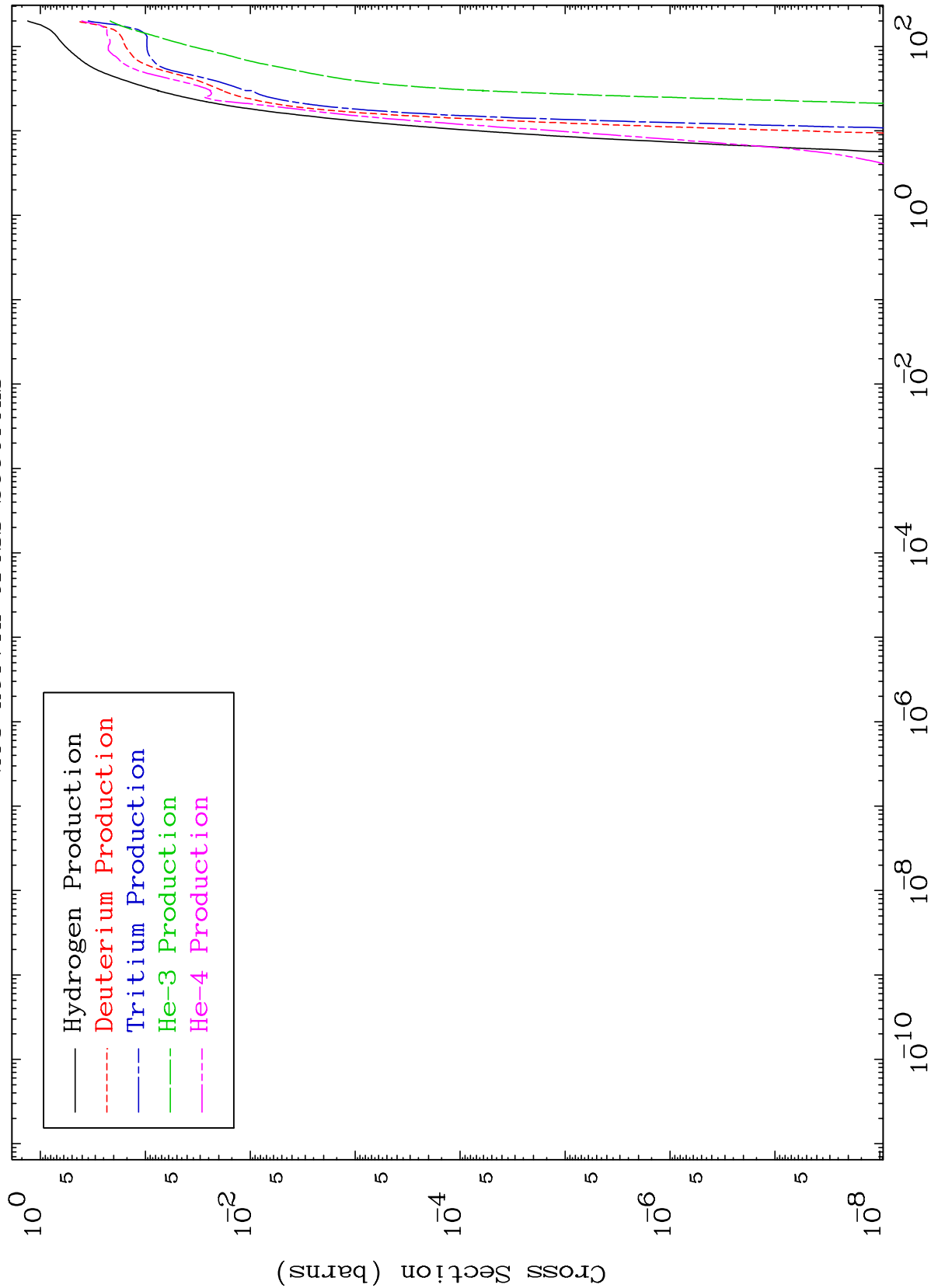




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Particle Production
293 Kelvin Cross Sections

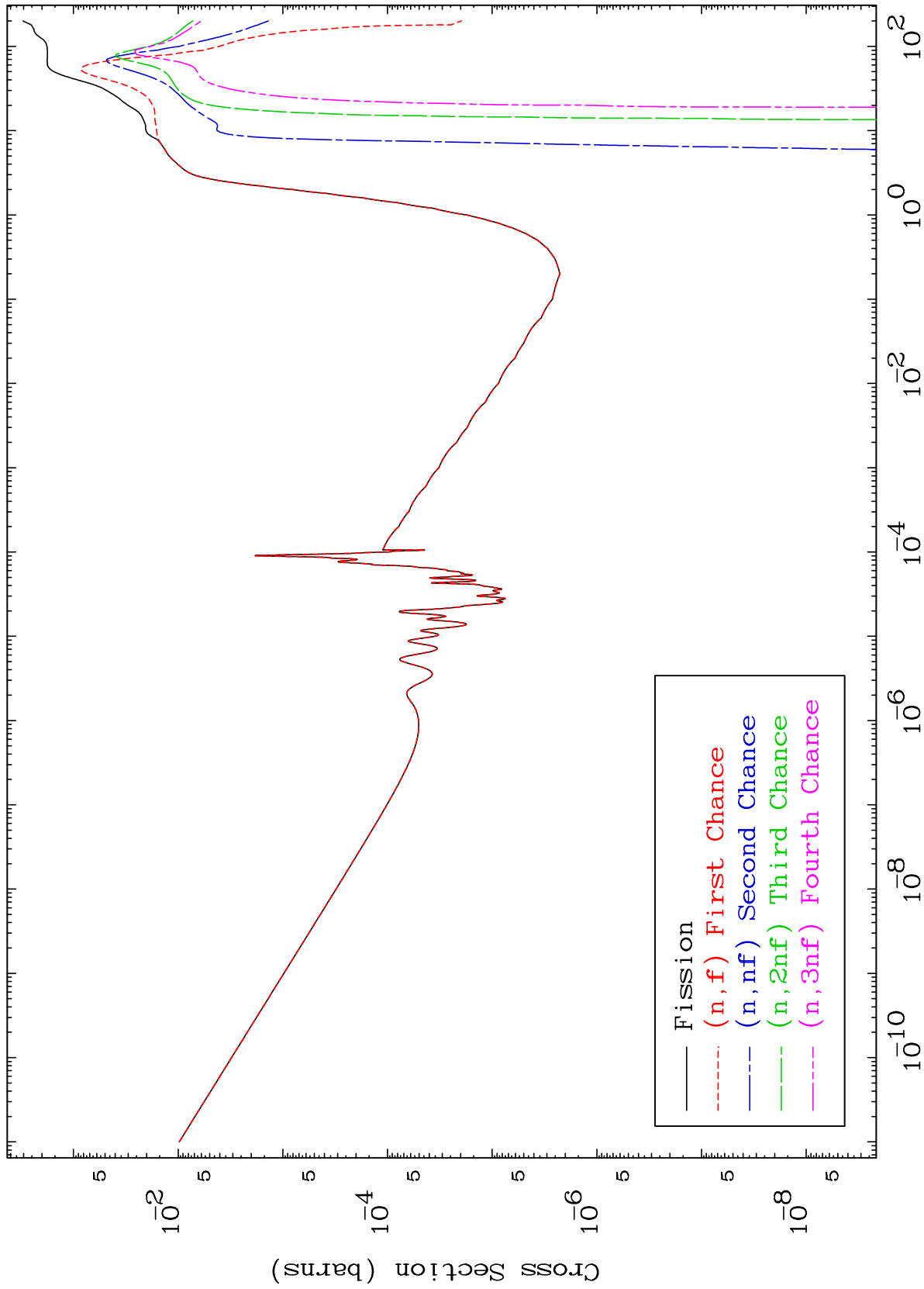
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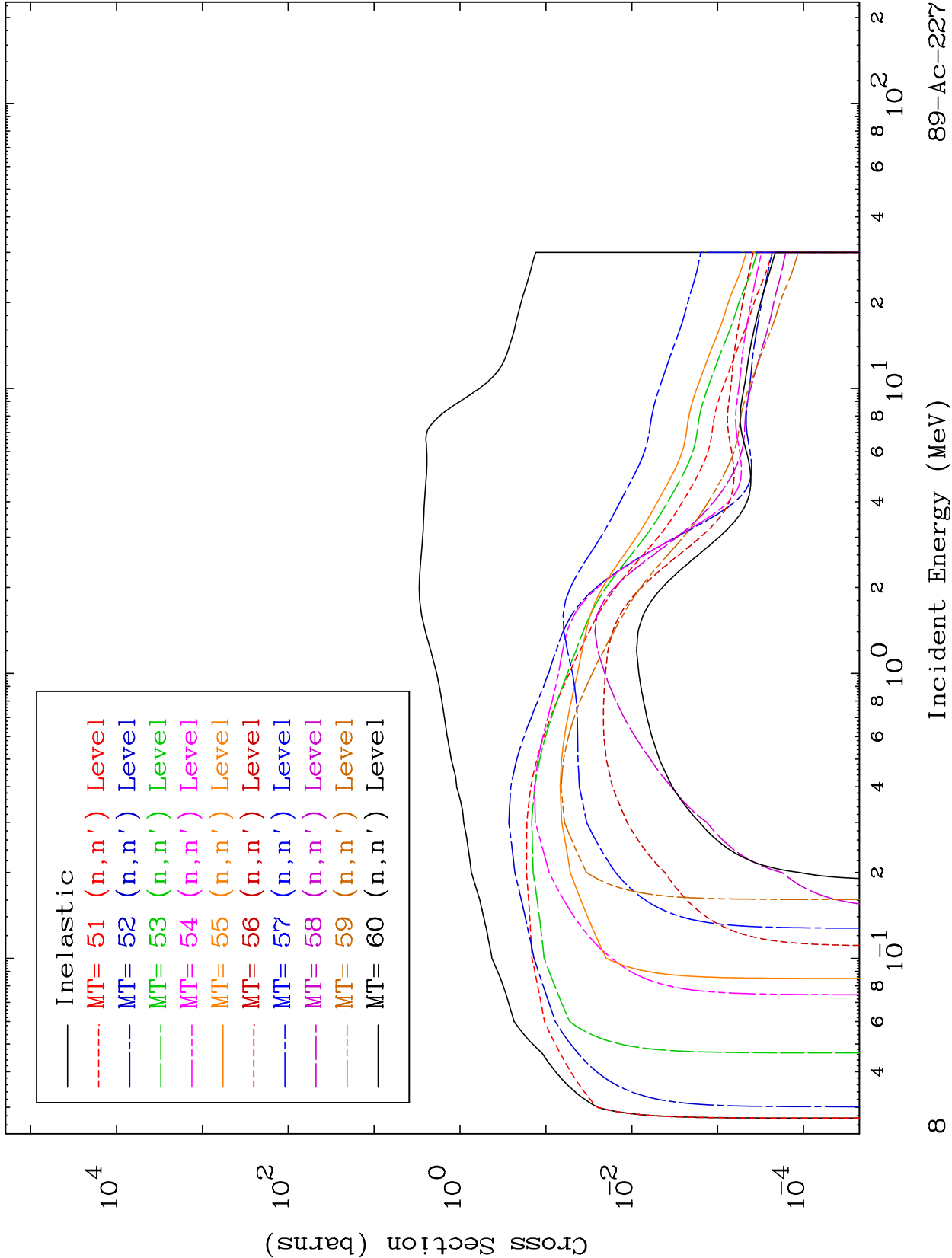


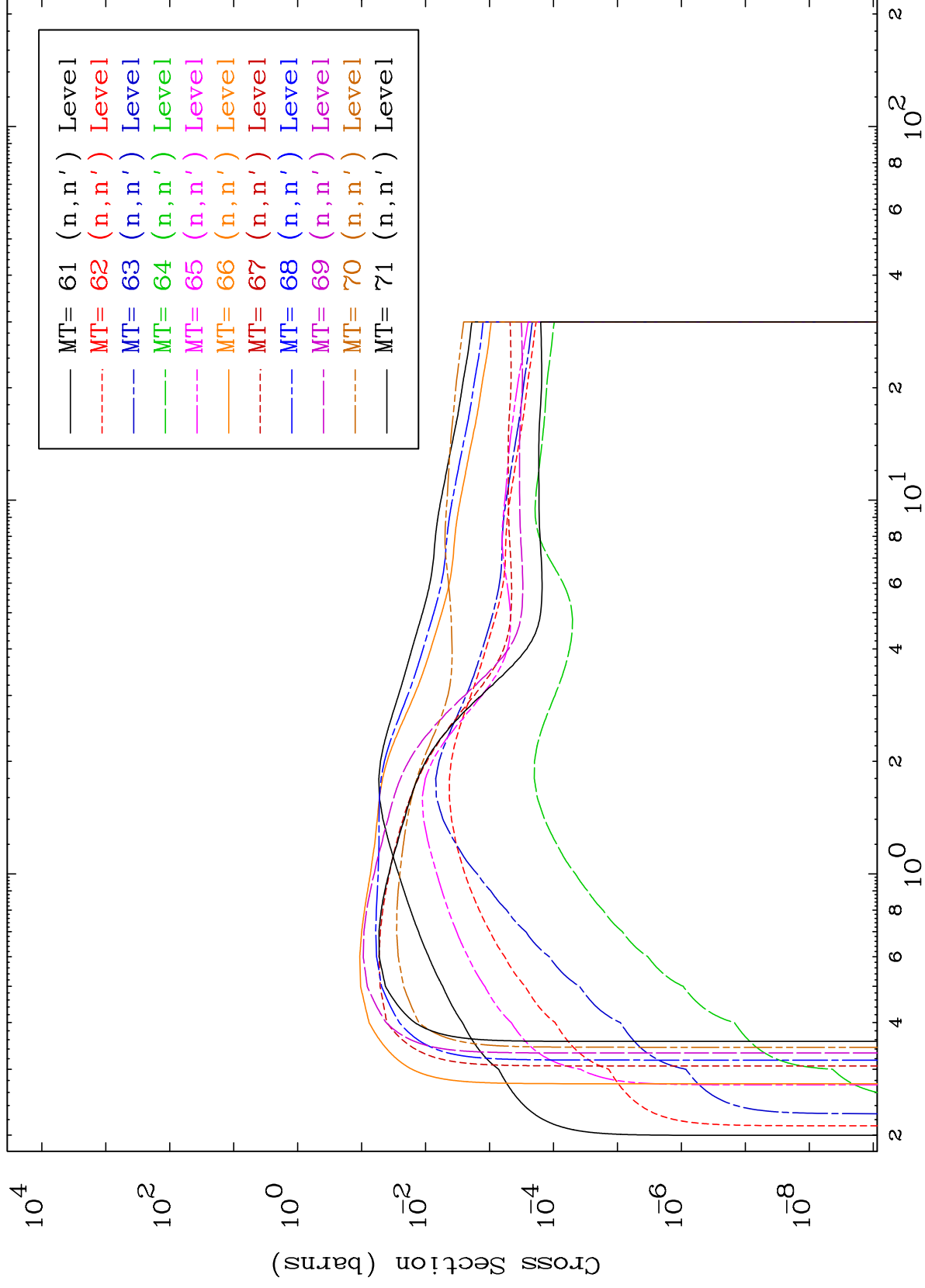
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Fission
293 Kelvin Cross Sections

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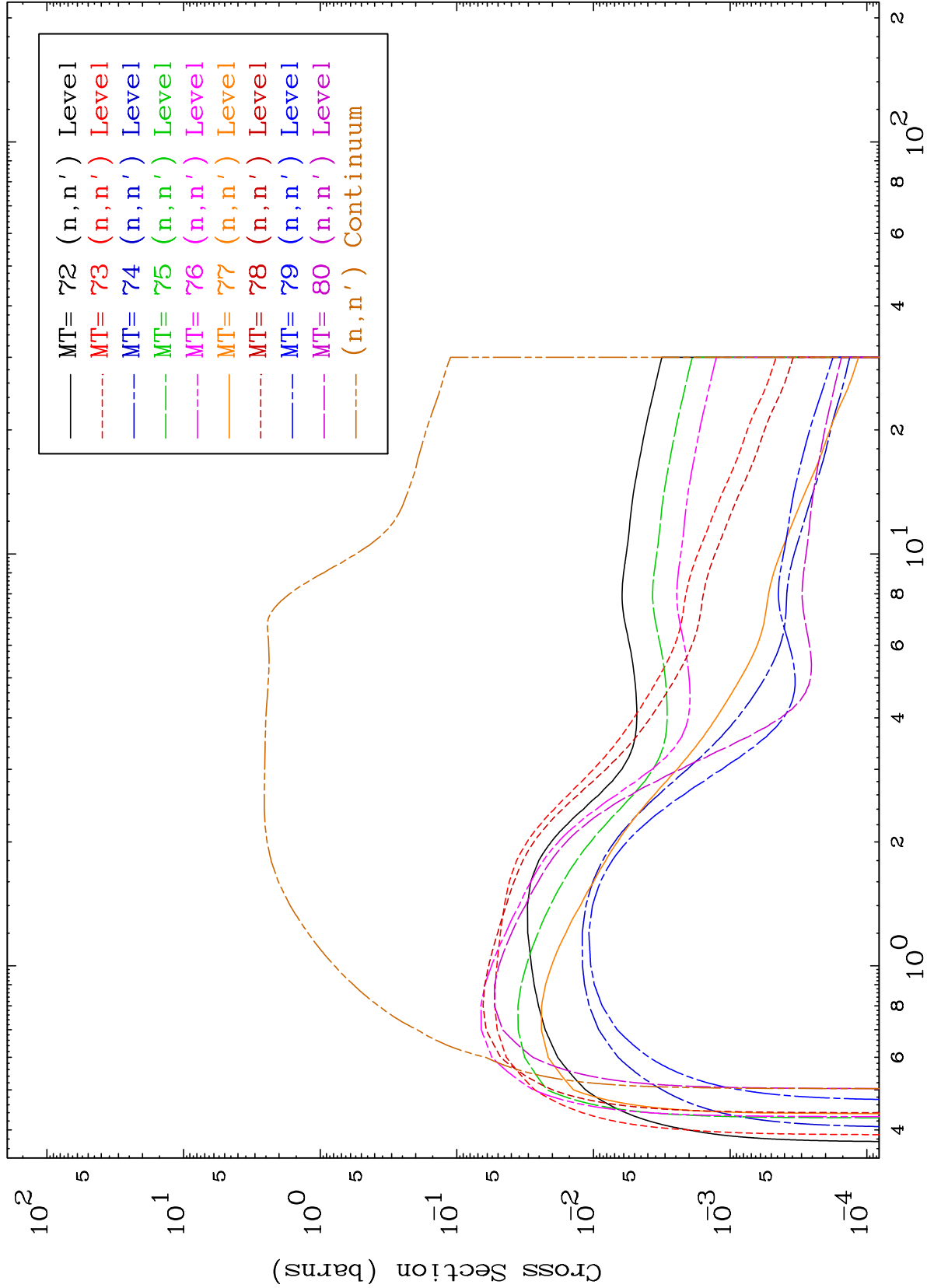


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(n,n') Level

293 Kelvin Cross Sections

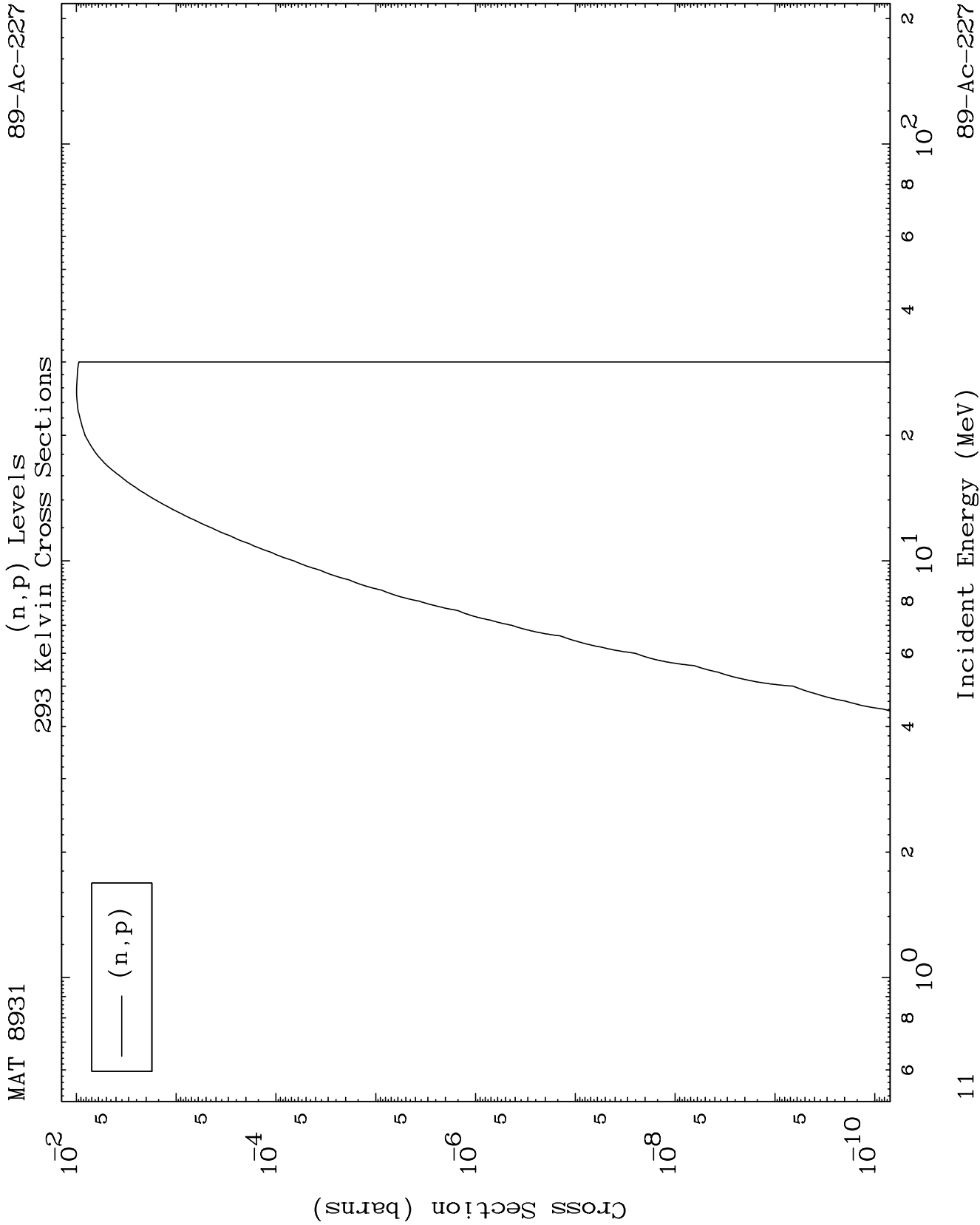
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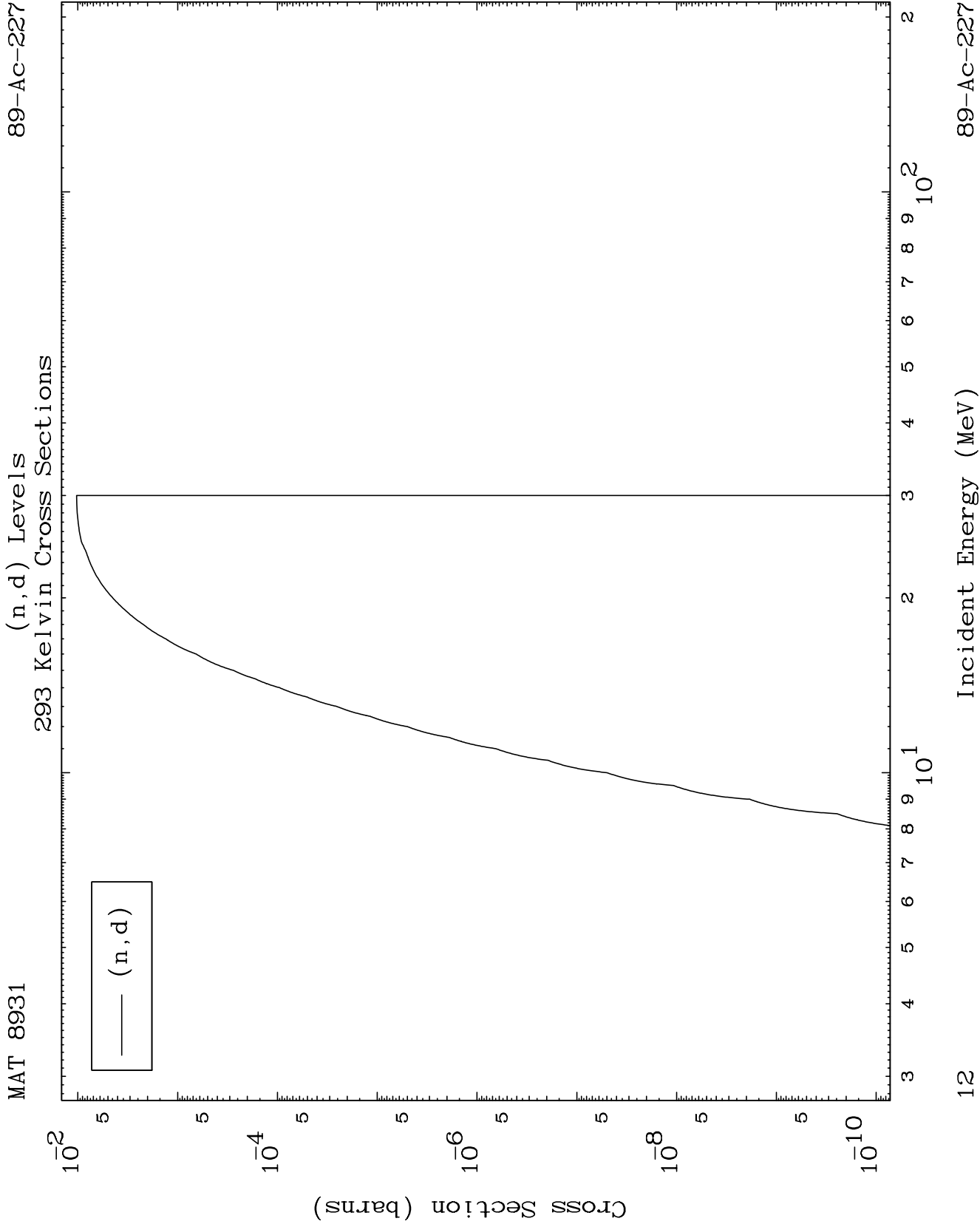


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Incident Energy (MeV)

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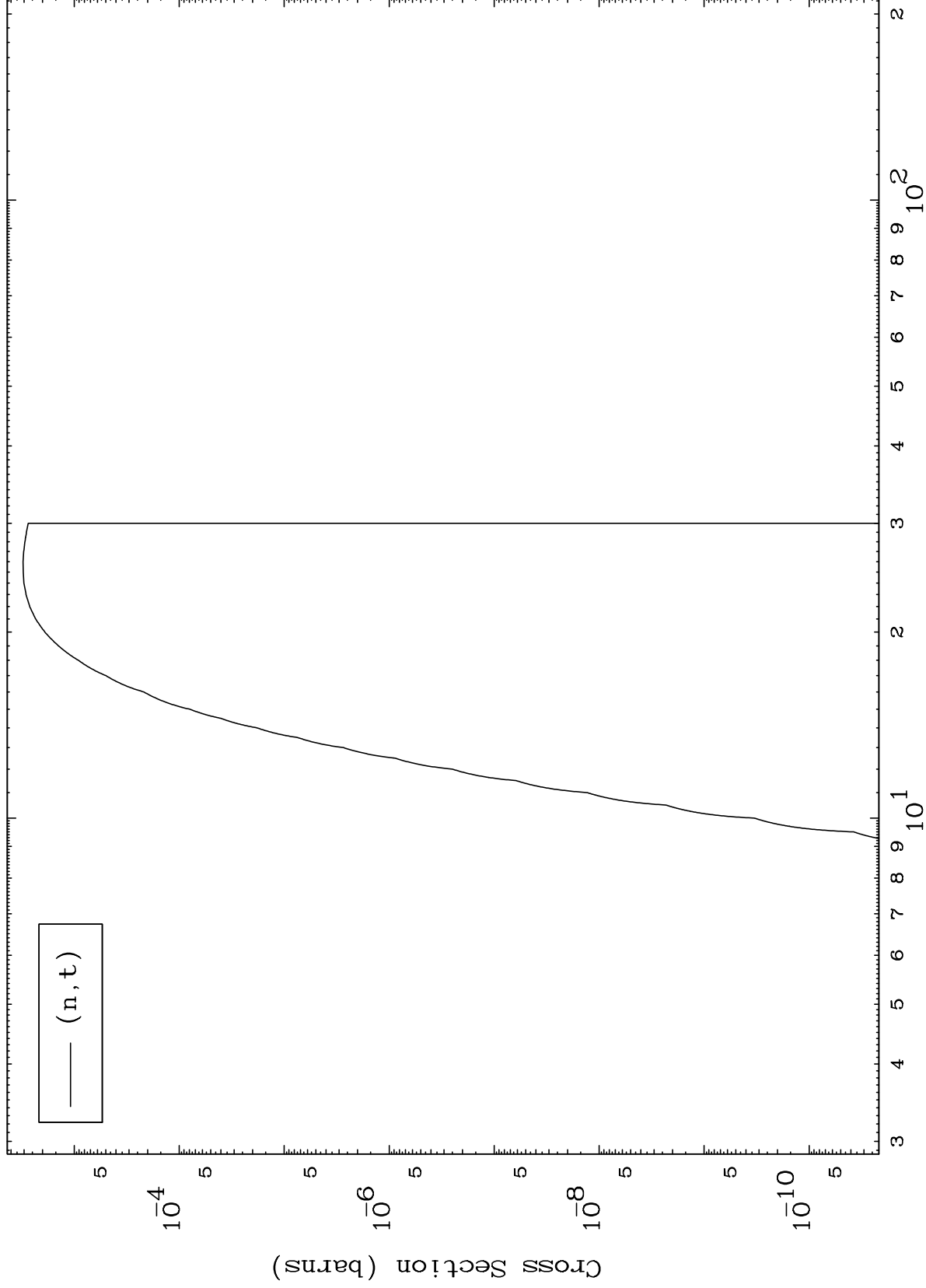




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(n,t) Levels
293 Kelvin Cross Sections



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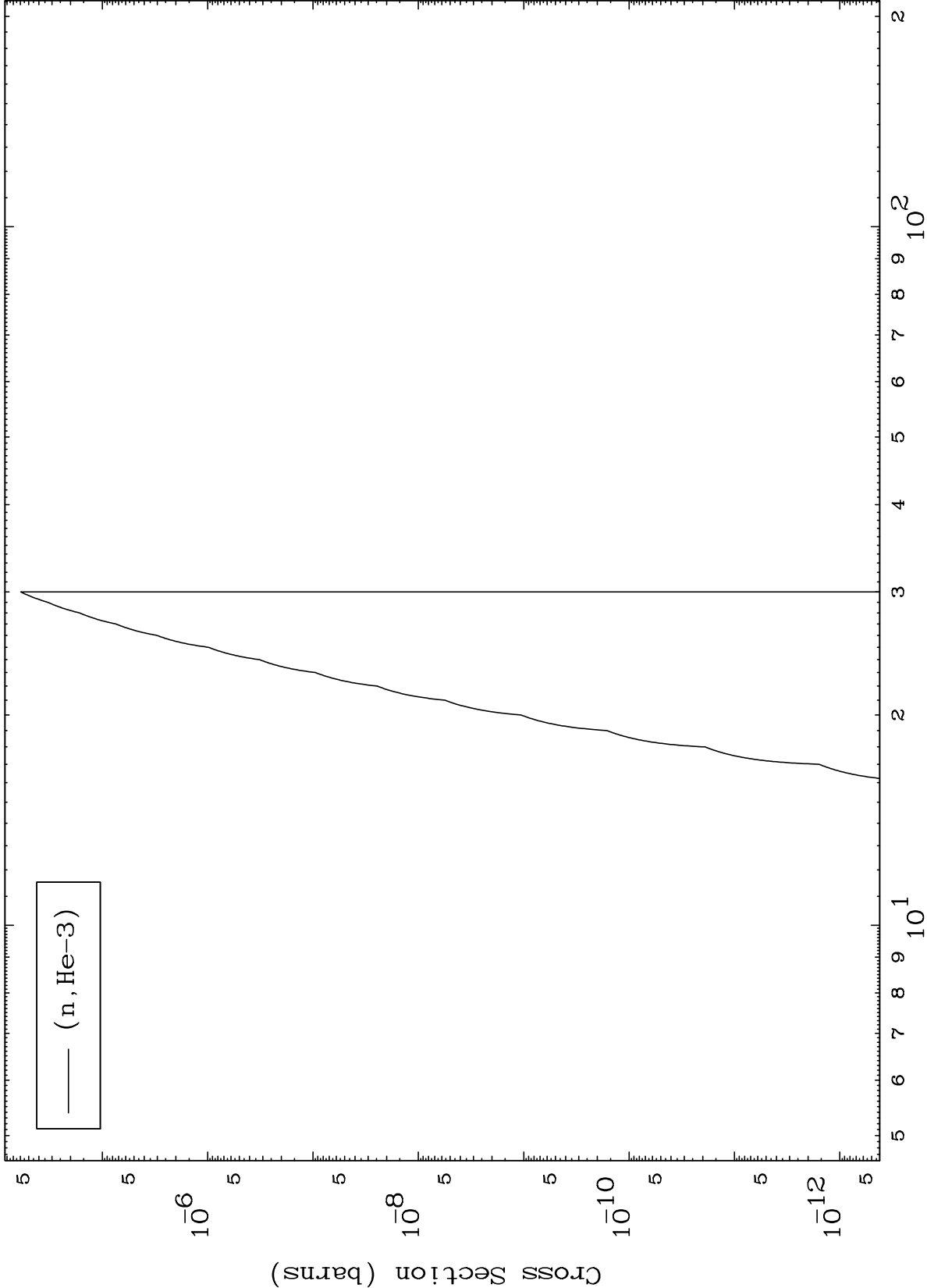
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

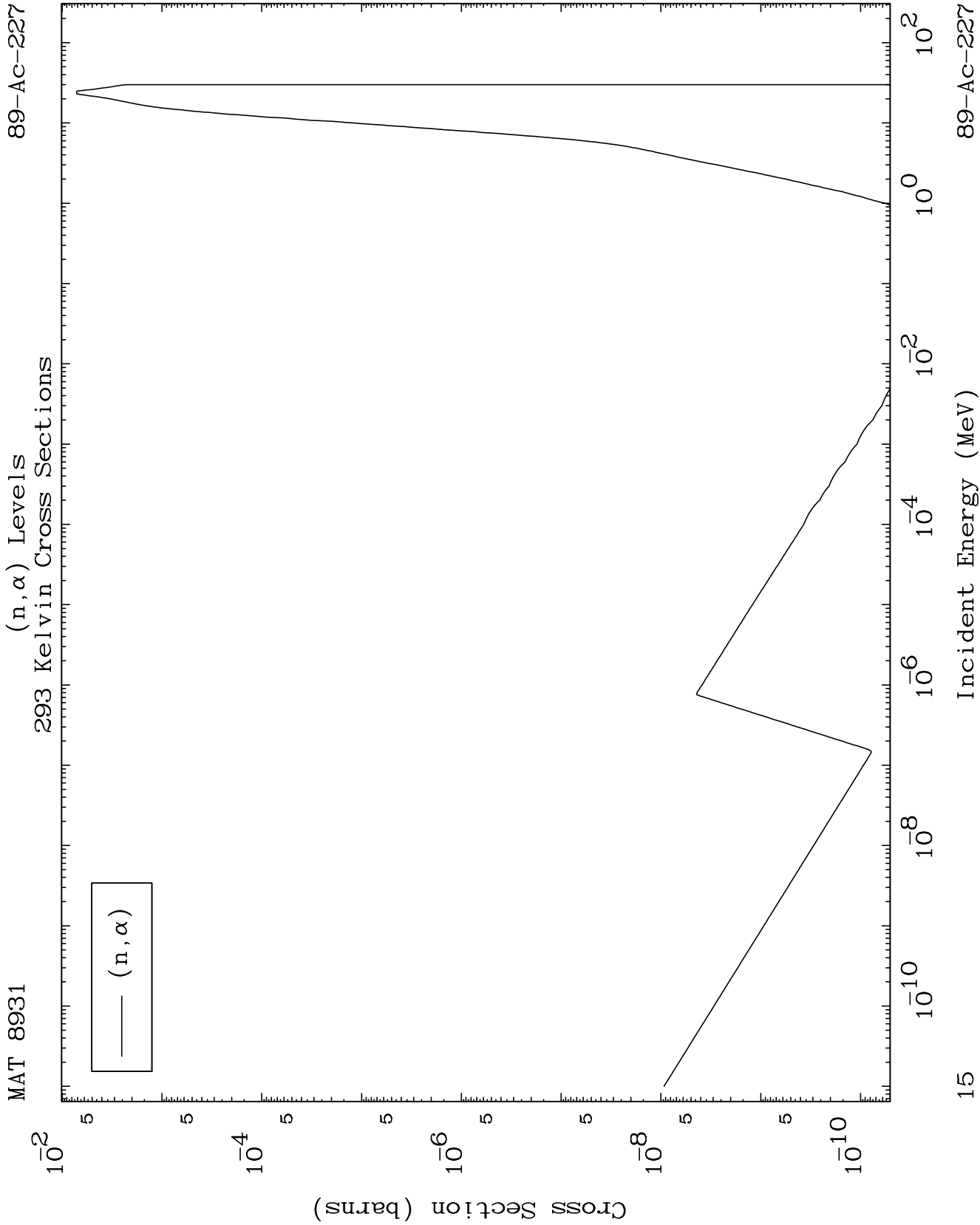
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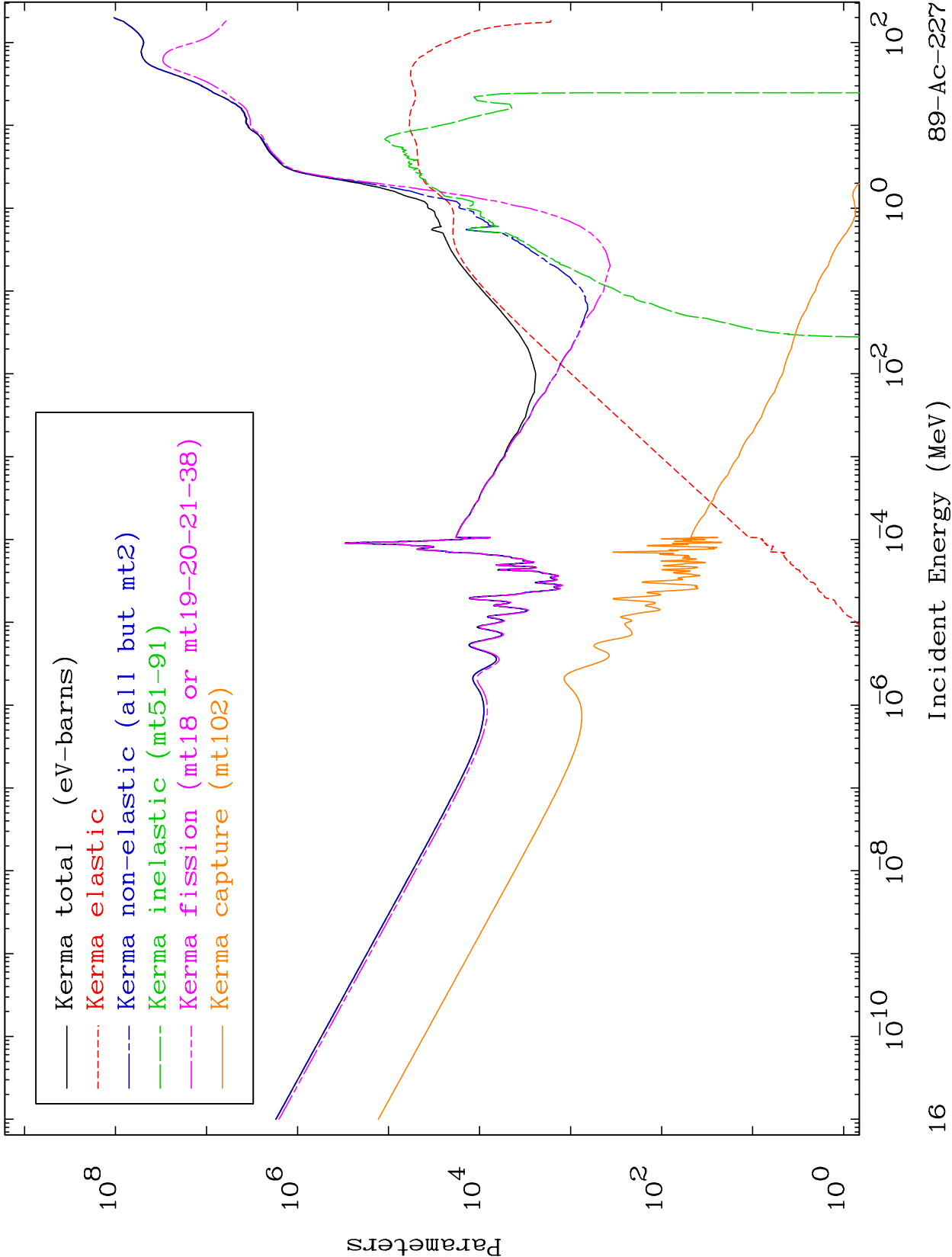


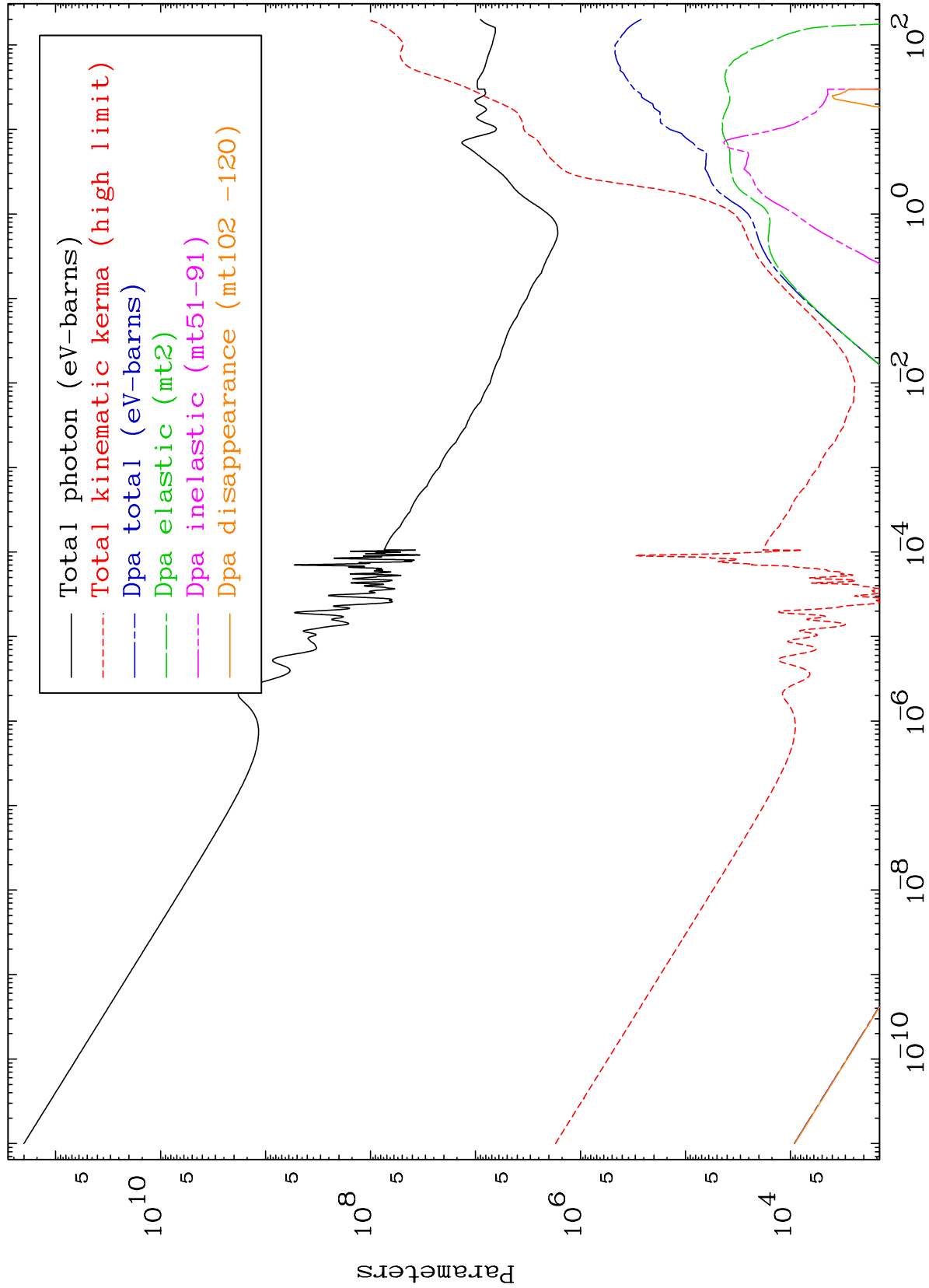
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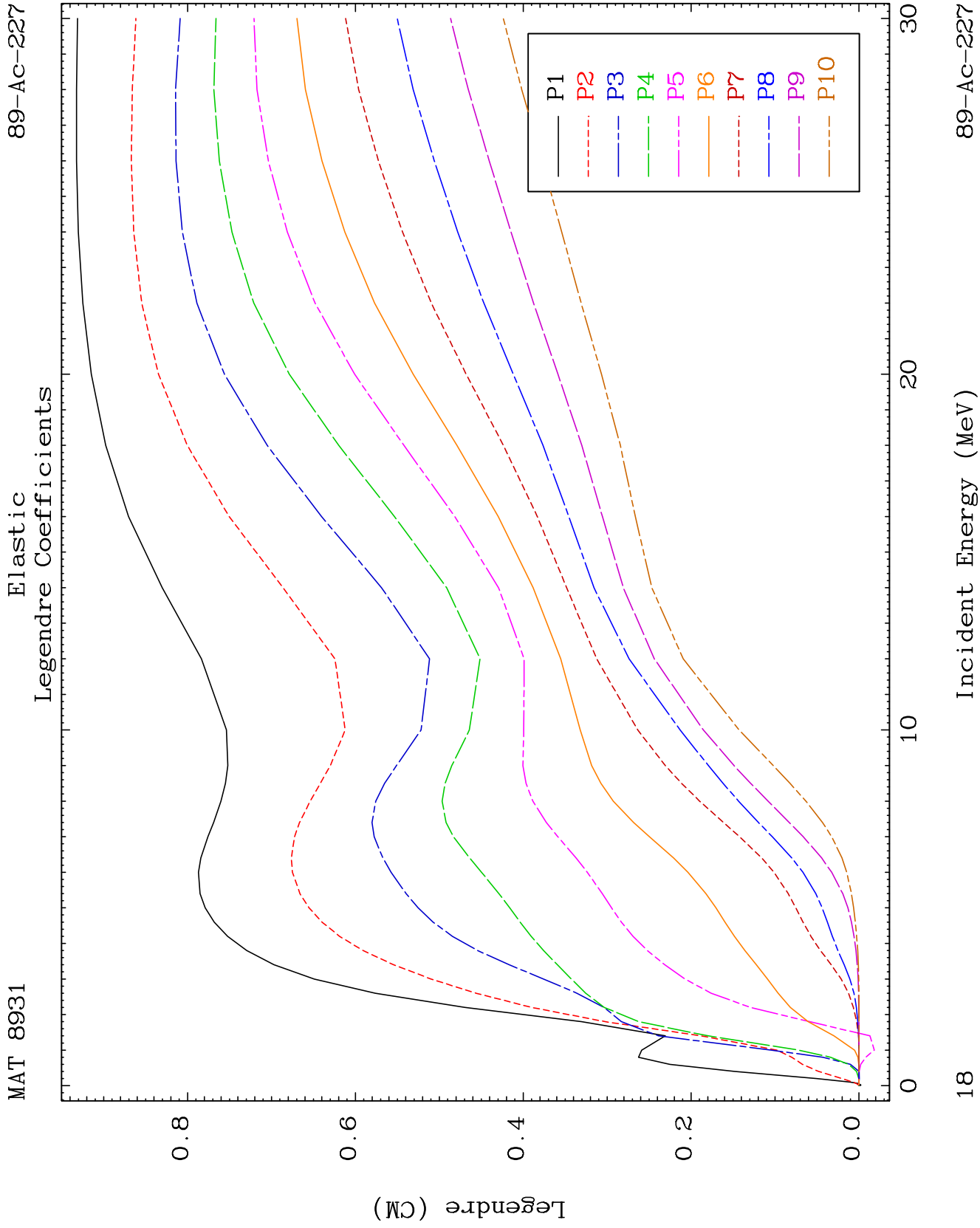
Incident Energy (MeV)

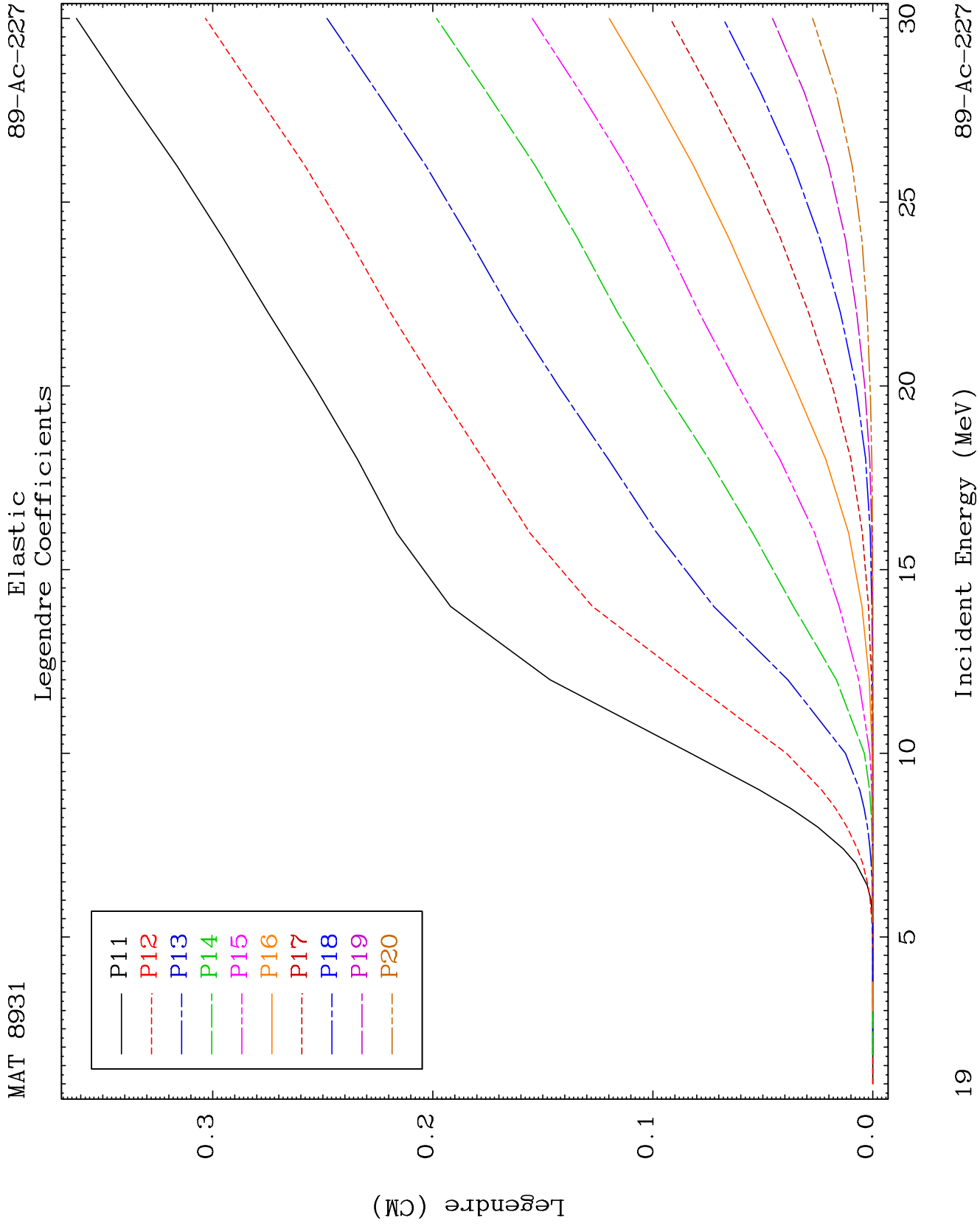
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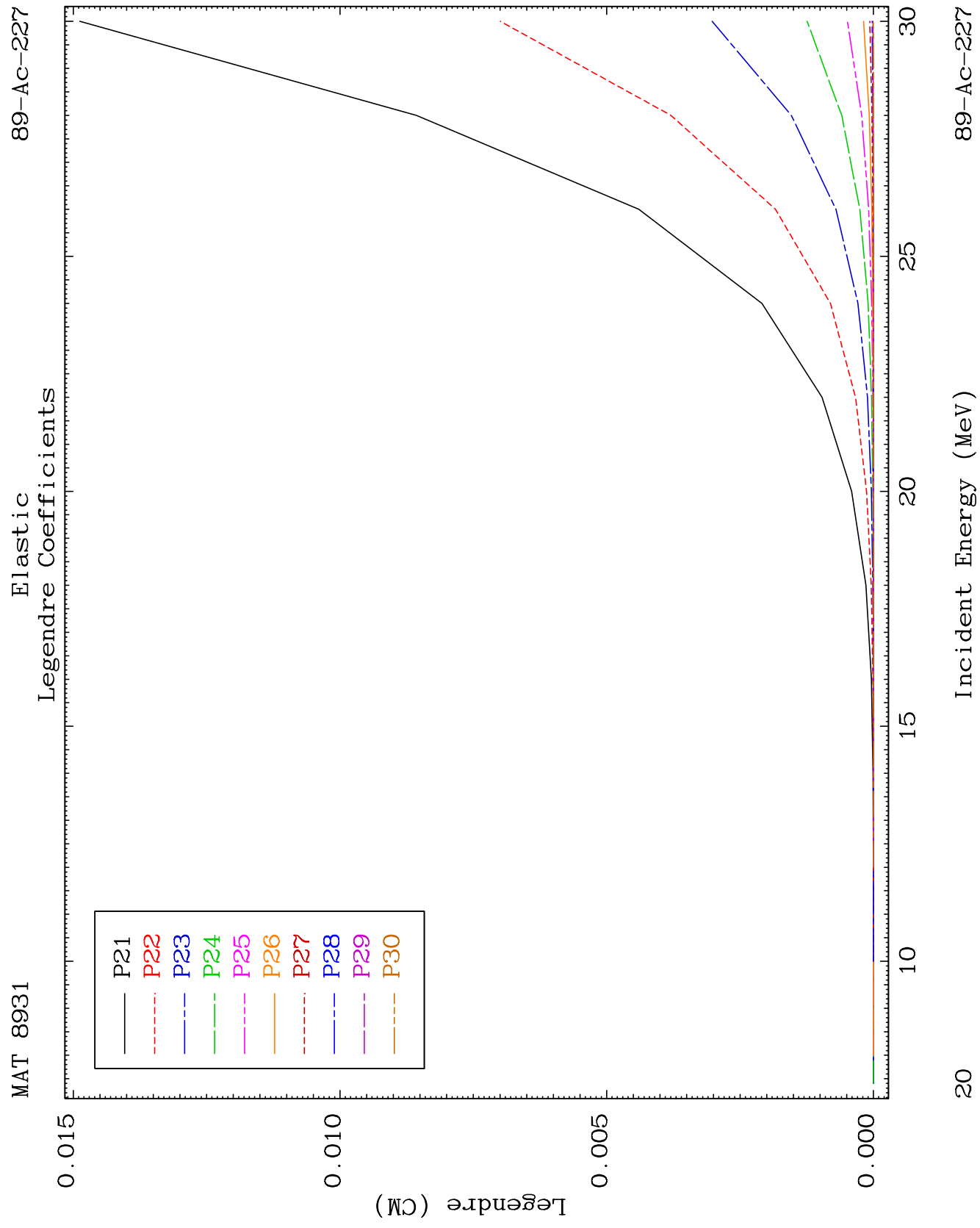


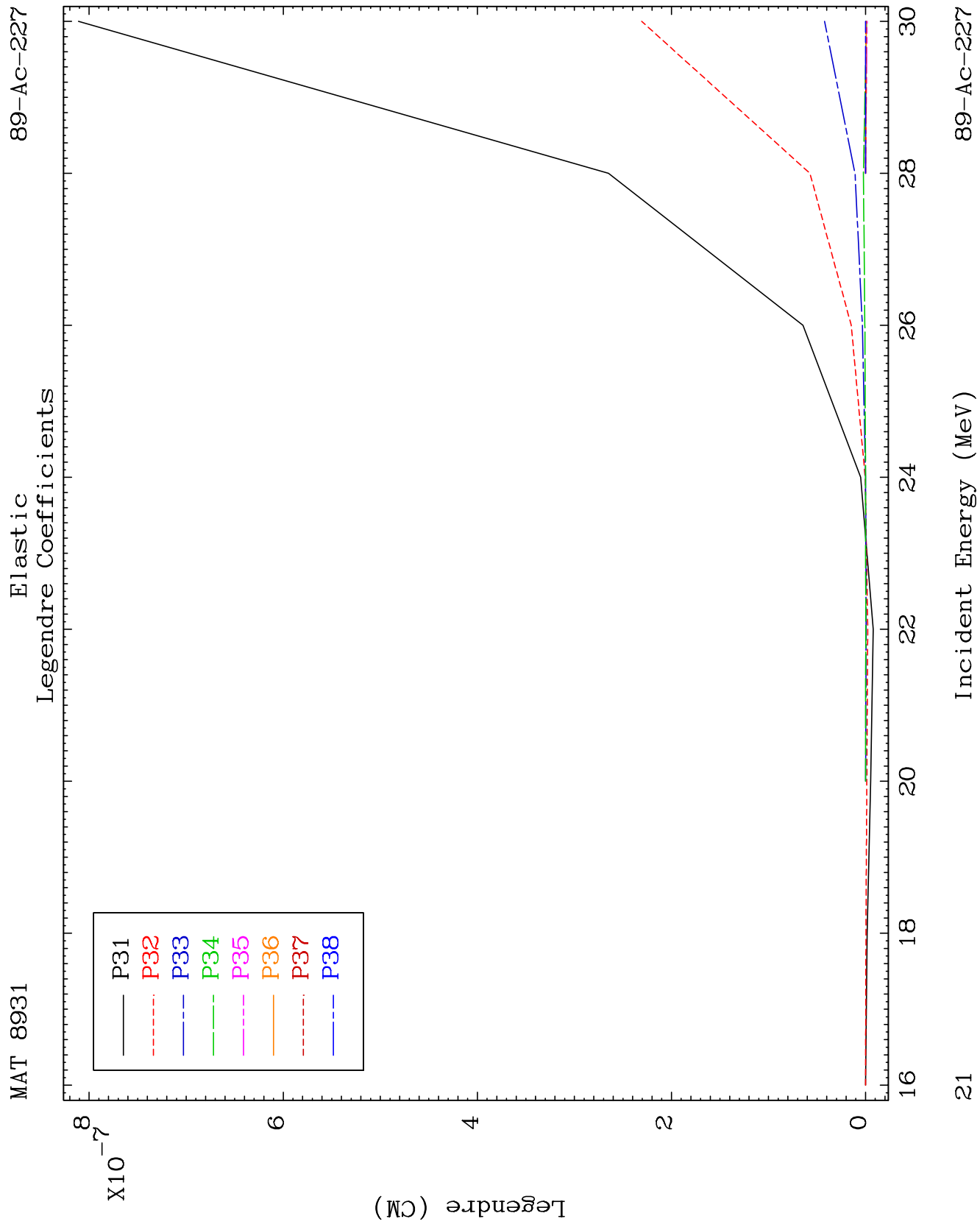


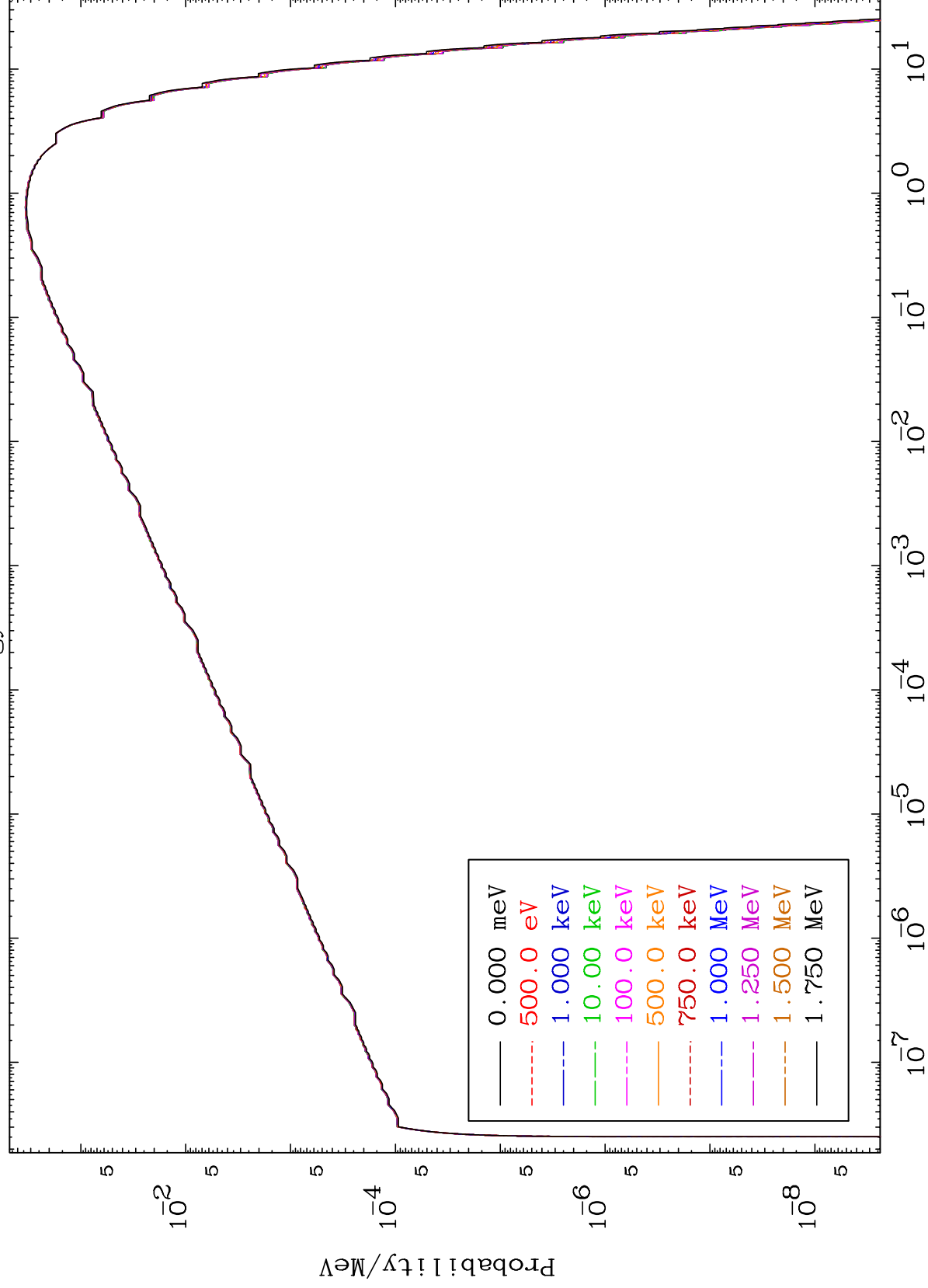








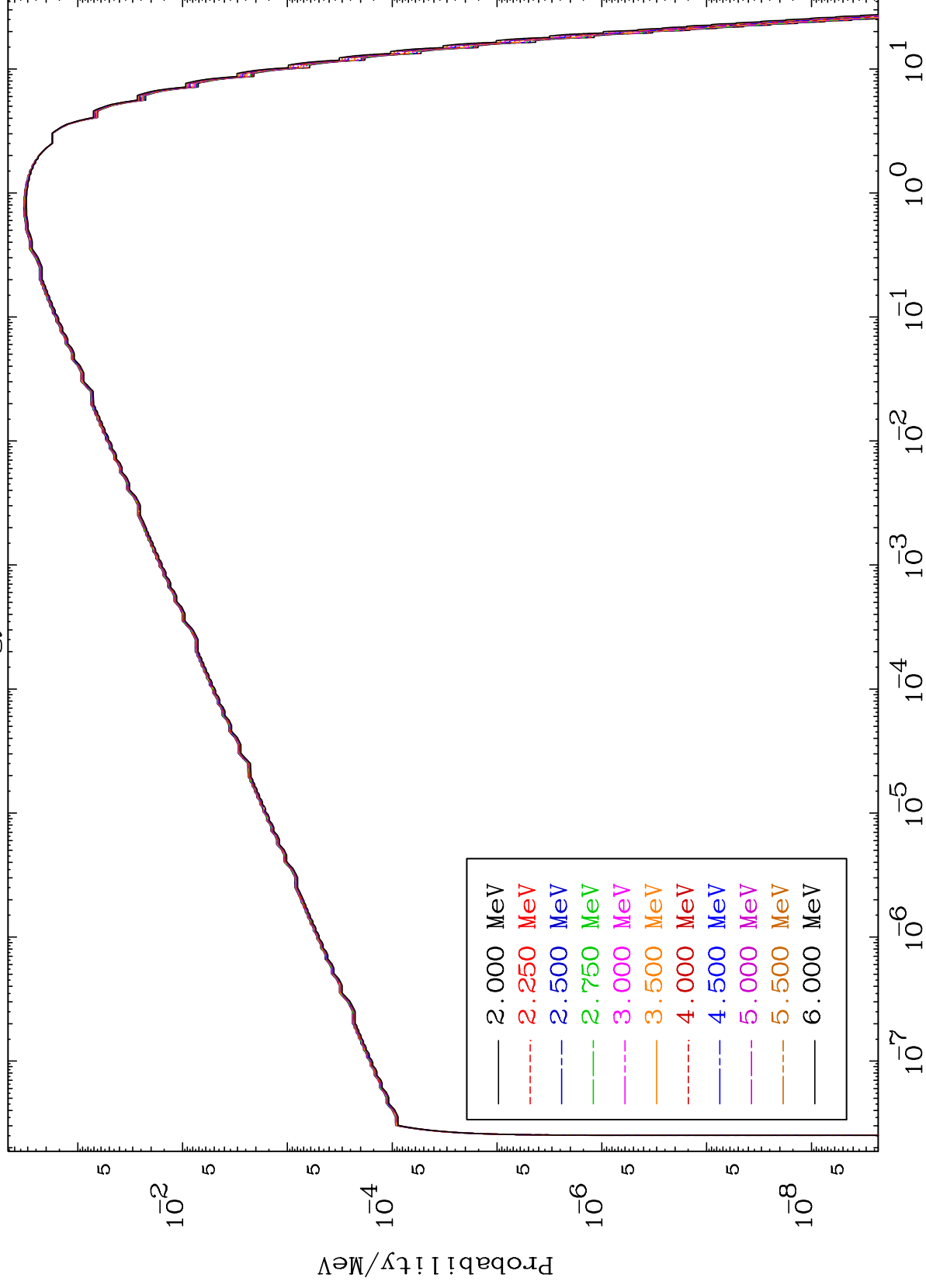


Fission
Energy Distribution

MAT 8931

Fission
Energy Distribution

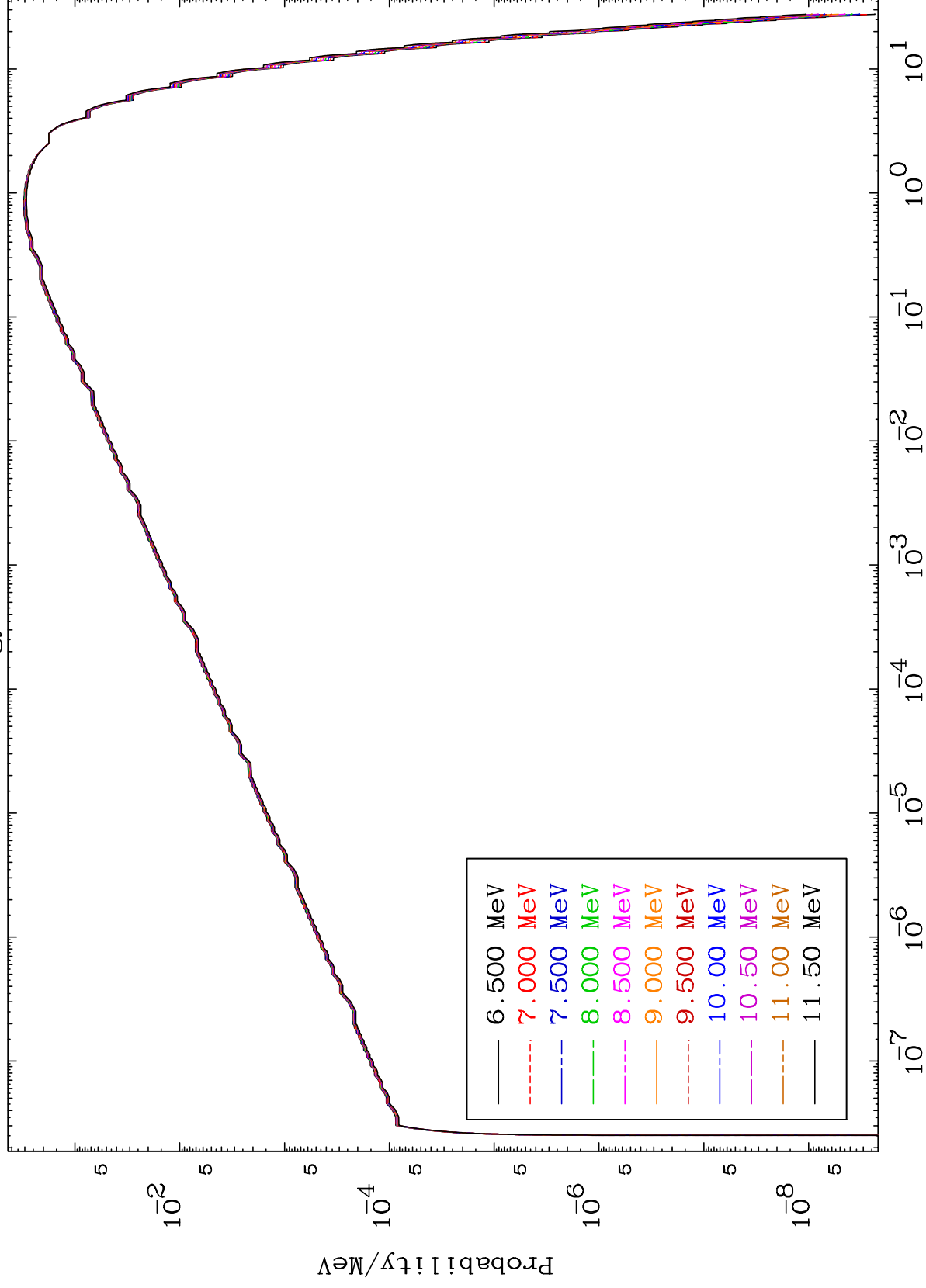
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Fission
Energy Distribution

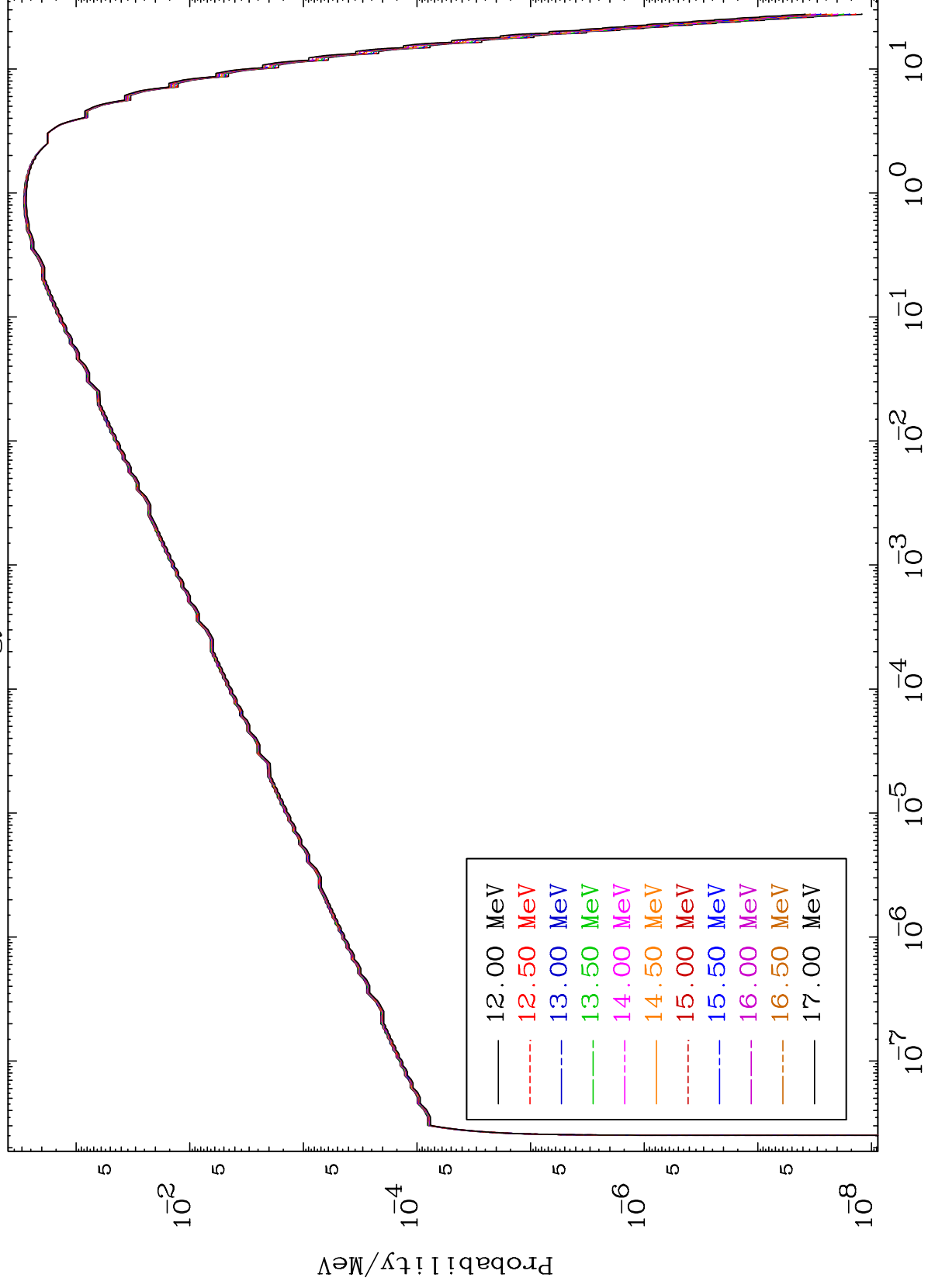
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Fission
Energy Distribution

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Fission
Energy Distribution