

Program EVALPLOT
(Version 2017-1)

by

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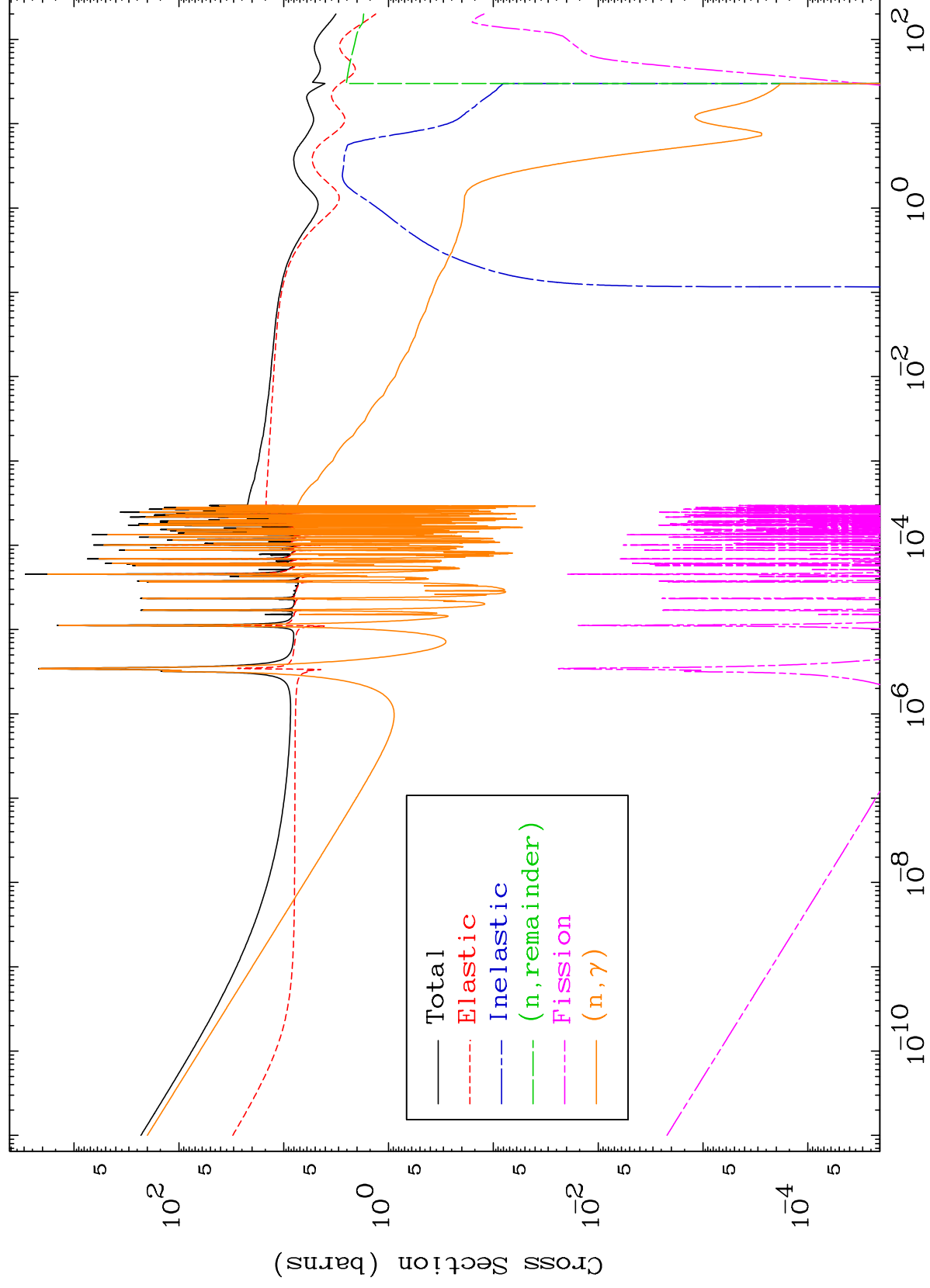
Press Mouse Button to Start

MAT 8591

Major

293 Kelvin Cross Sections

85-At-225



1

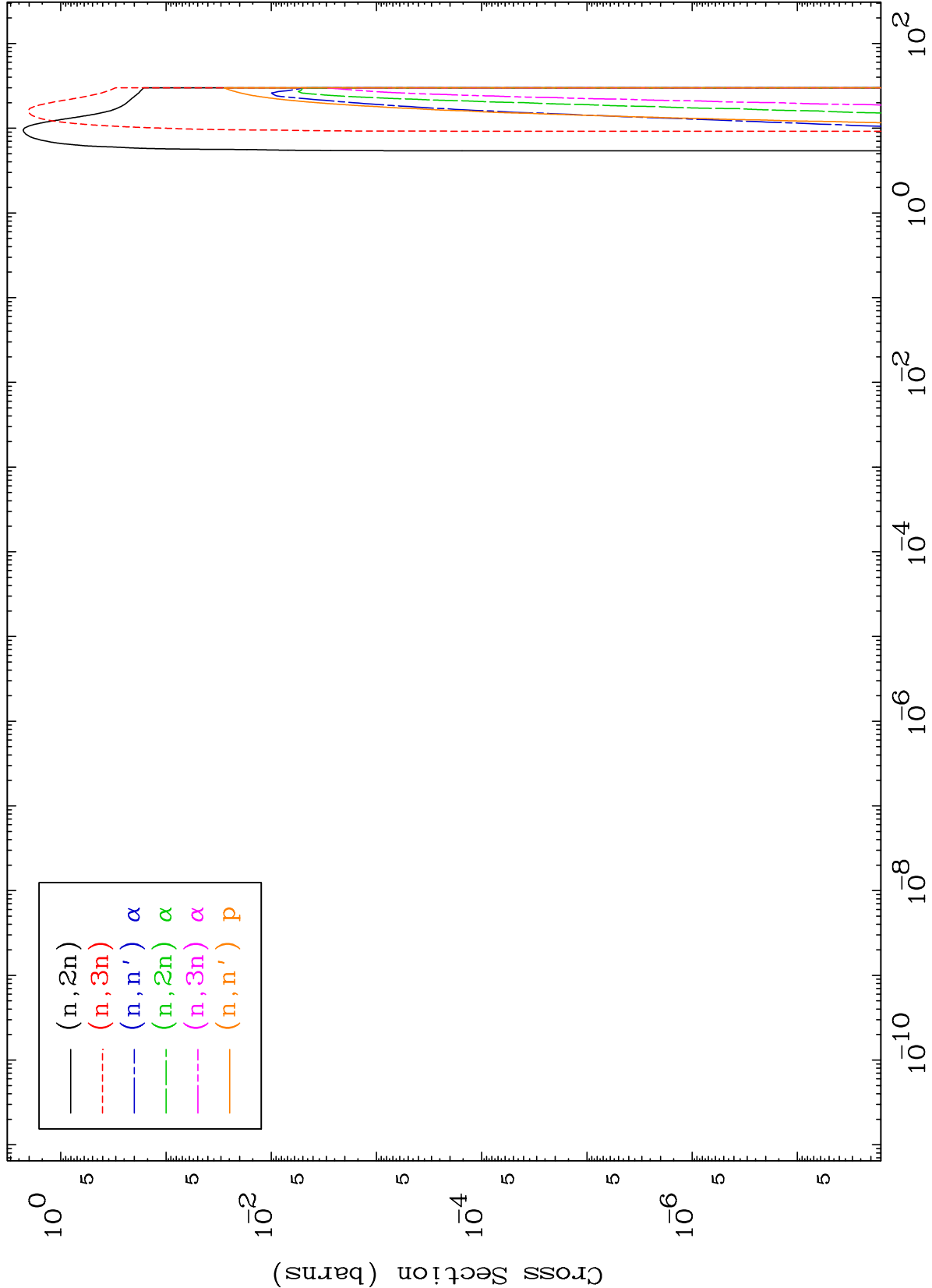
Incident Energy (MeV)

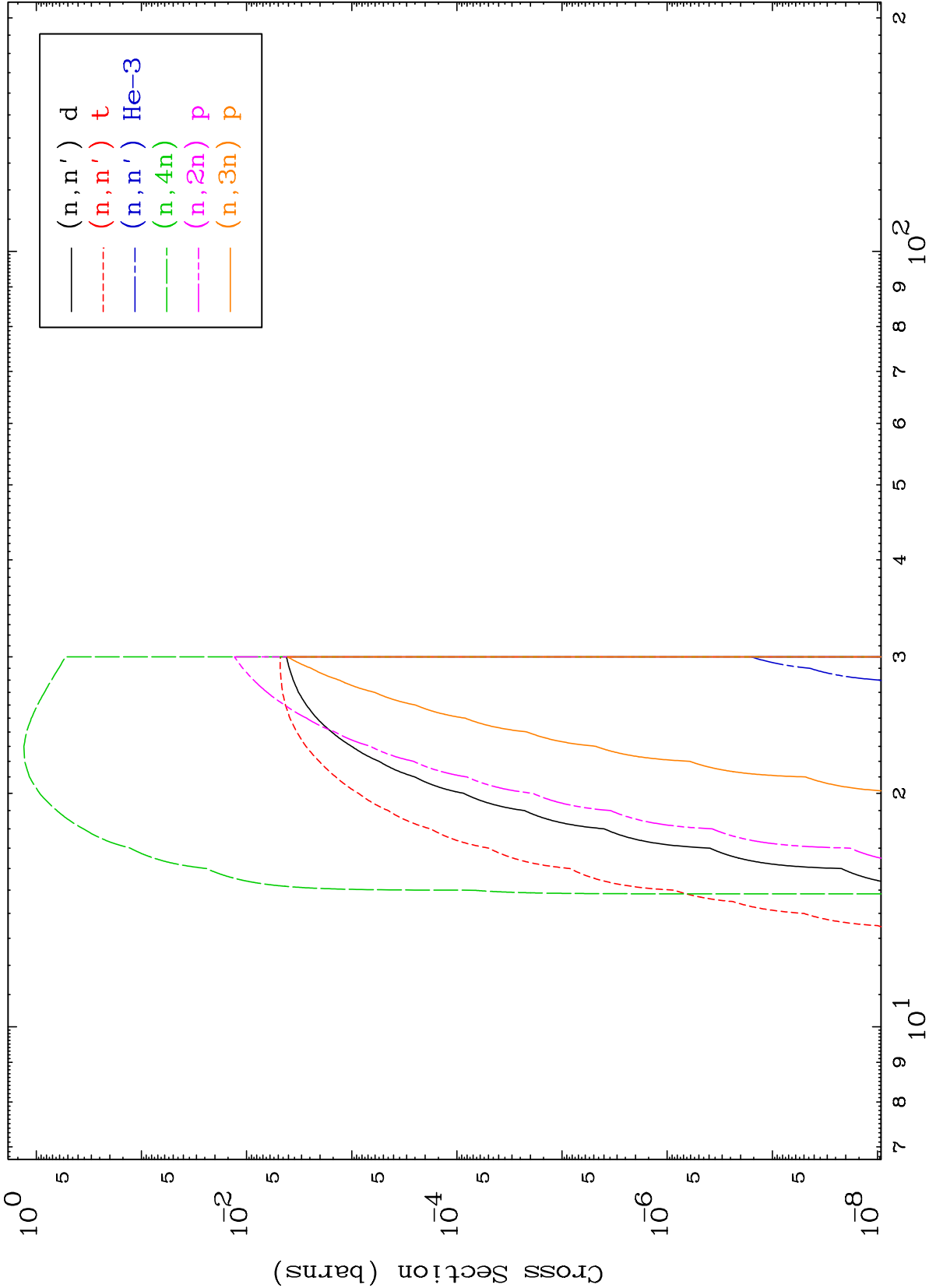
85-At-225

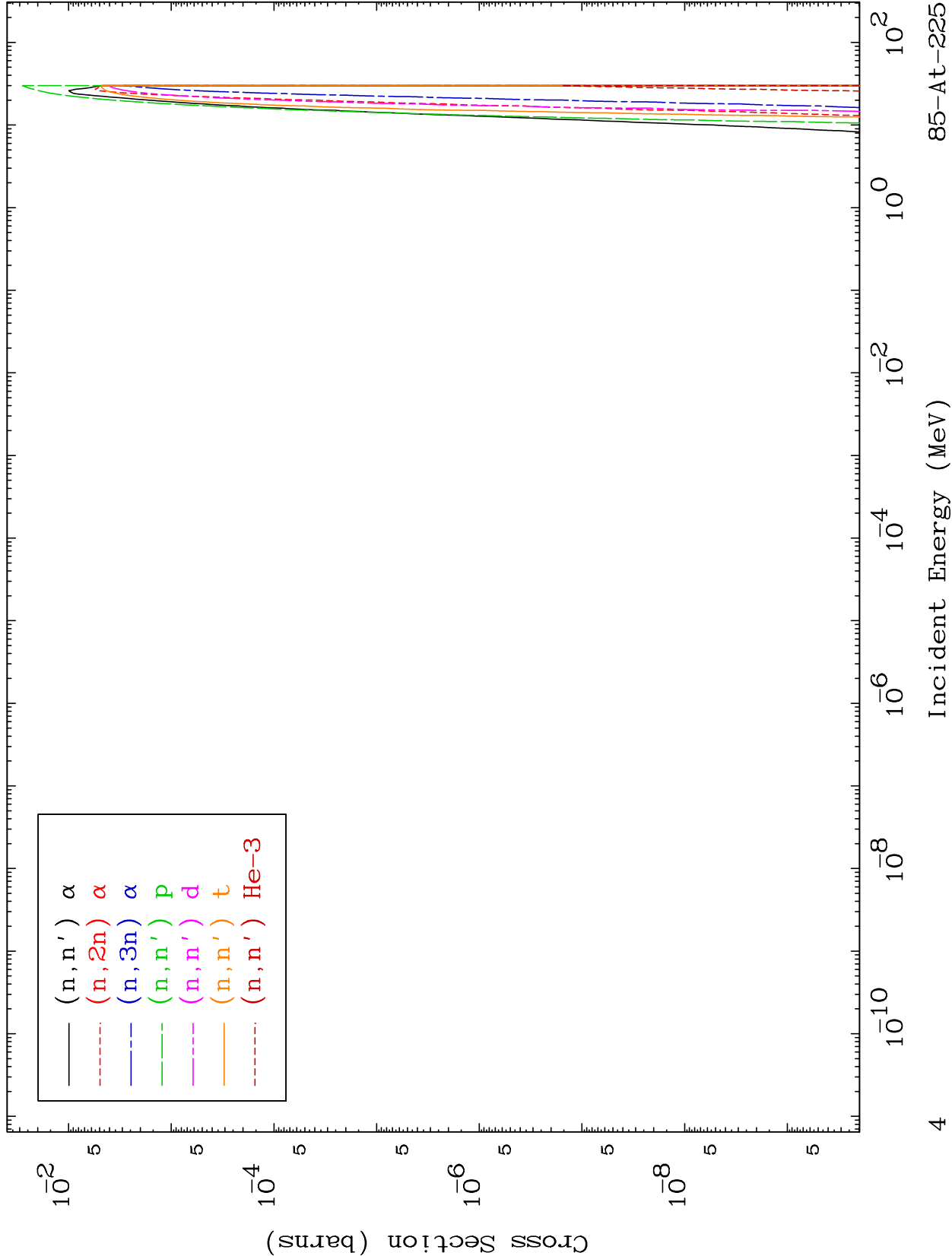
MAT 8591

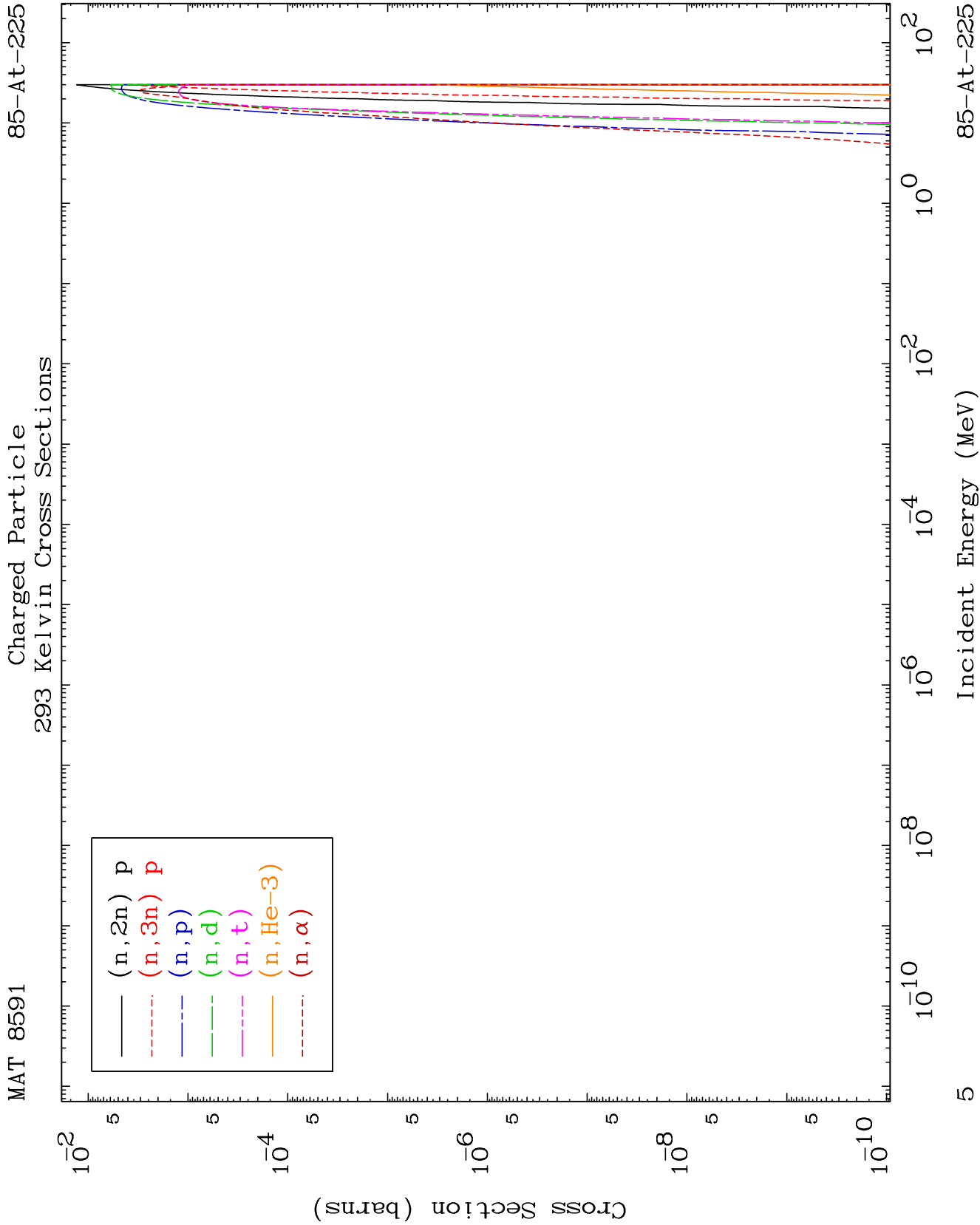
Neutron Production
293 Kelvin Cross Sections

85-At-225





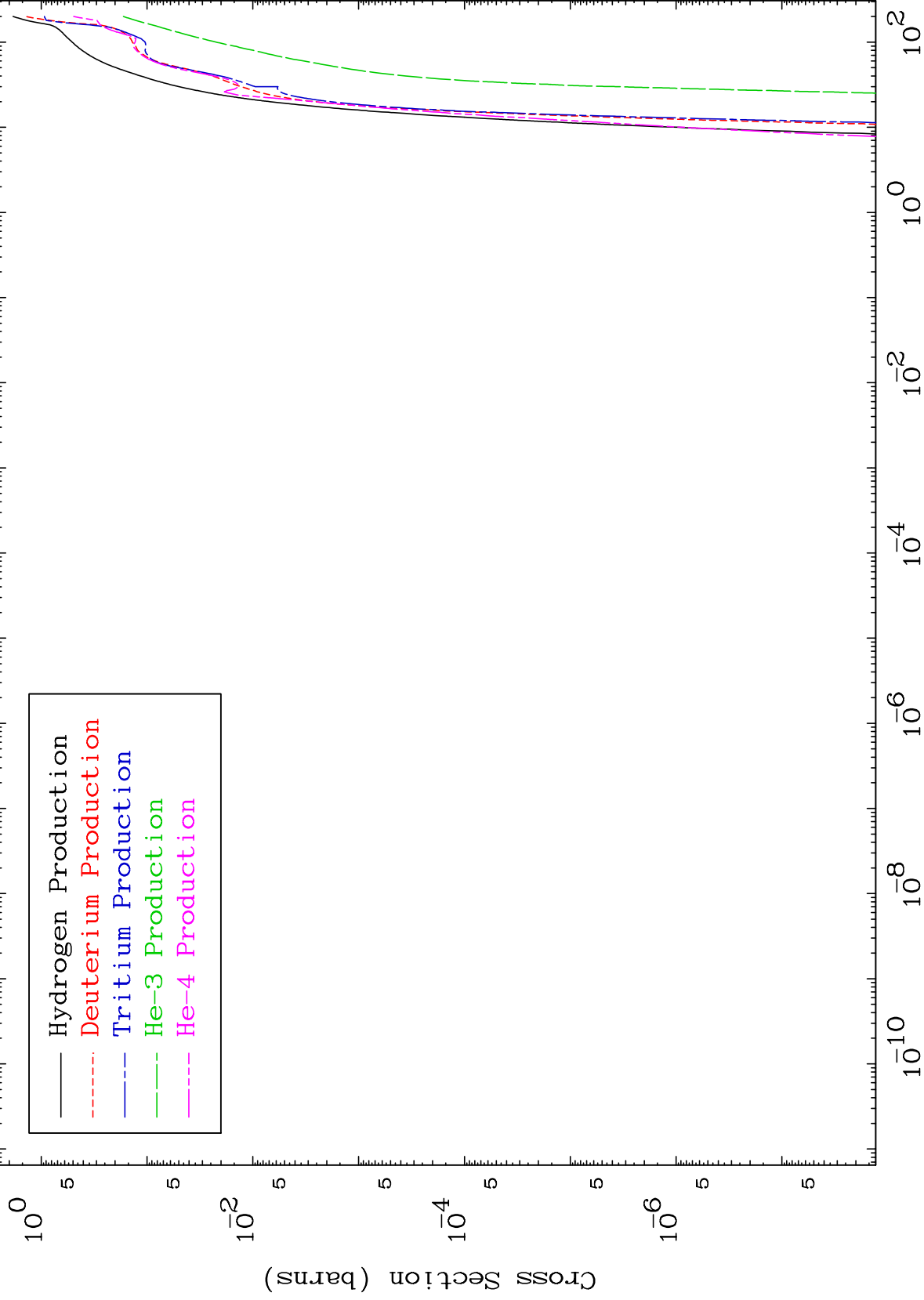


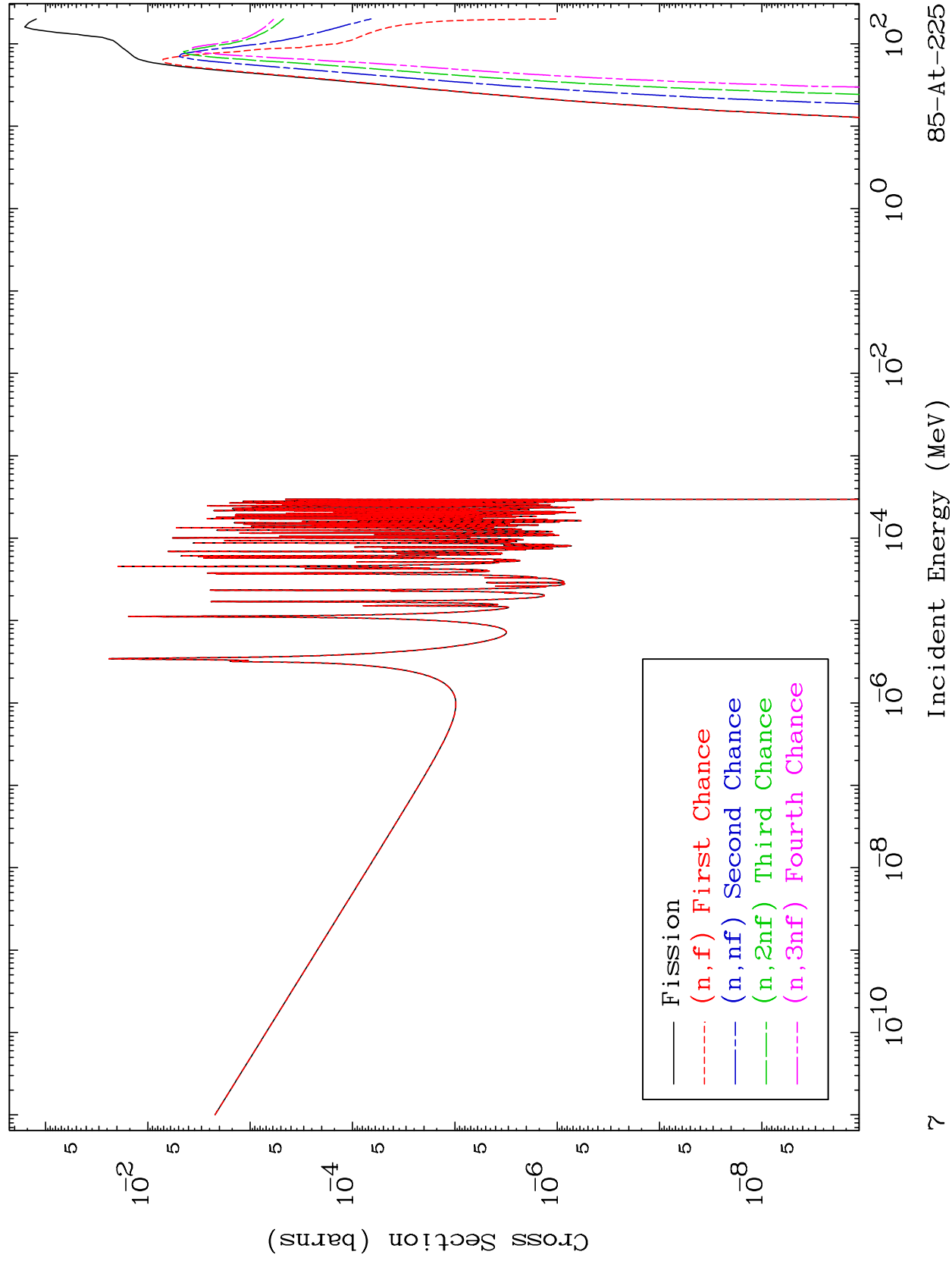


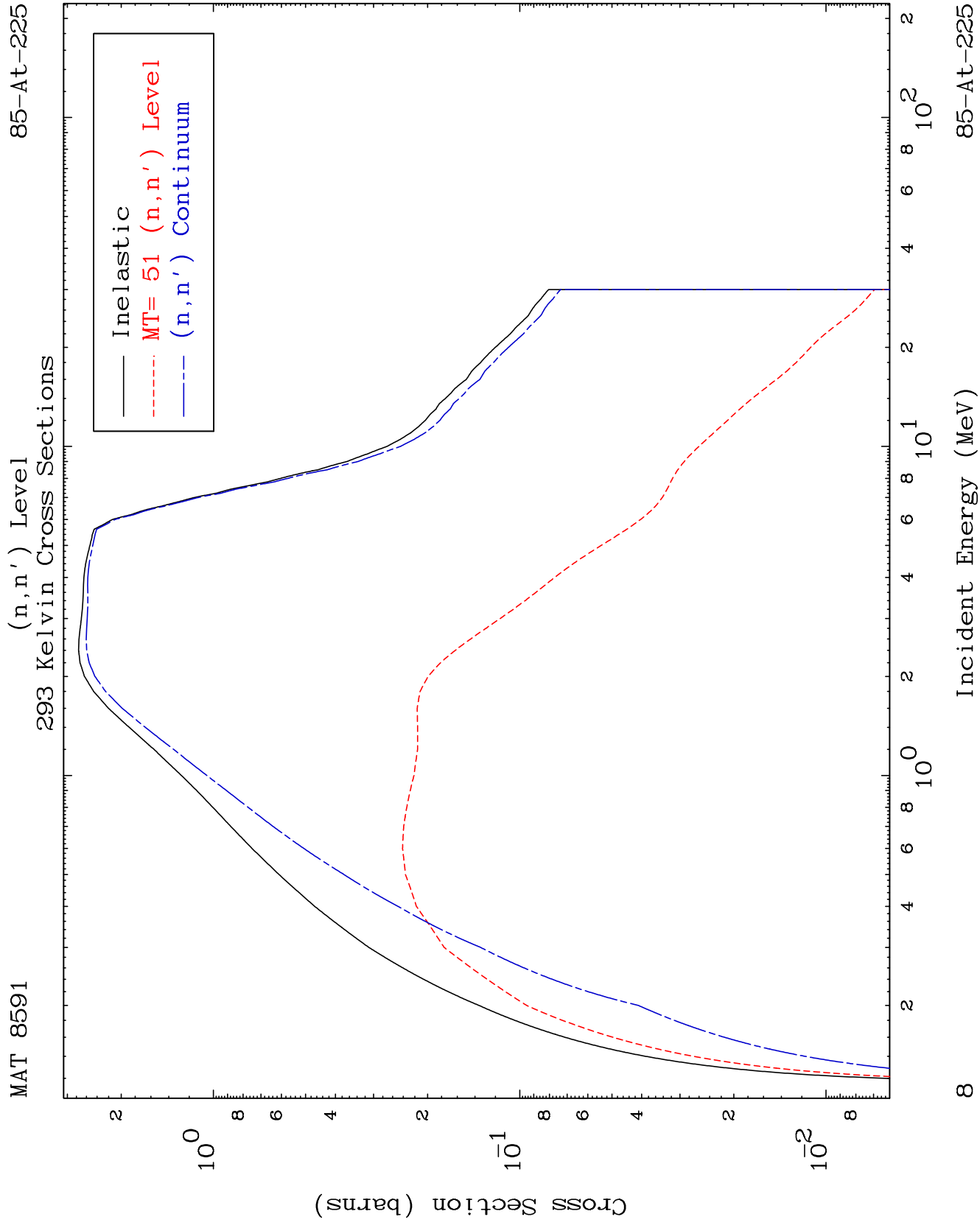
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Particle Production
293 Kelvin Cross Sections

85-At-225



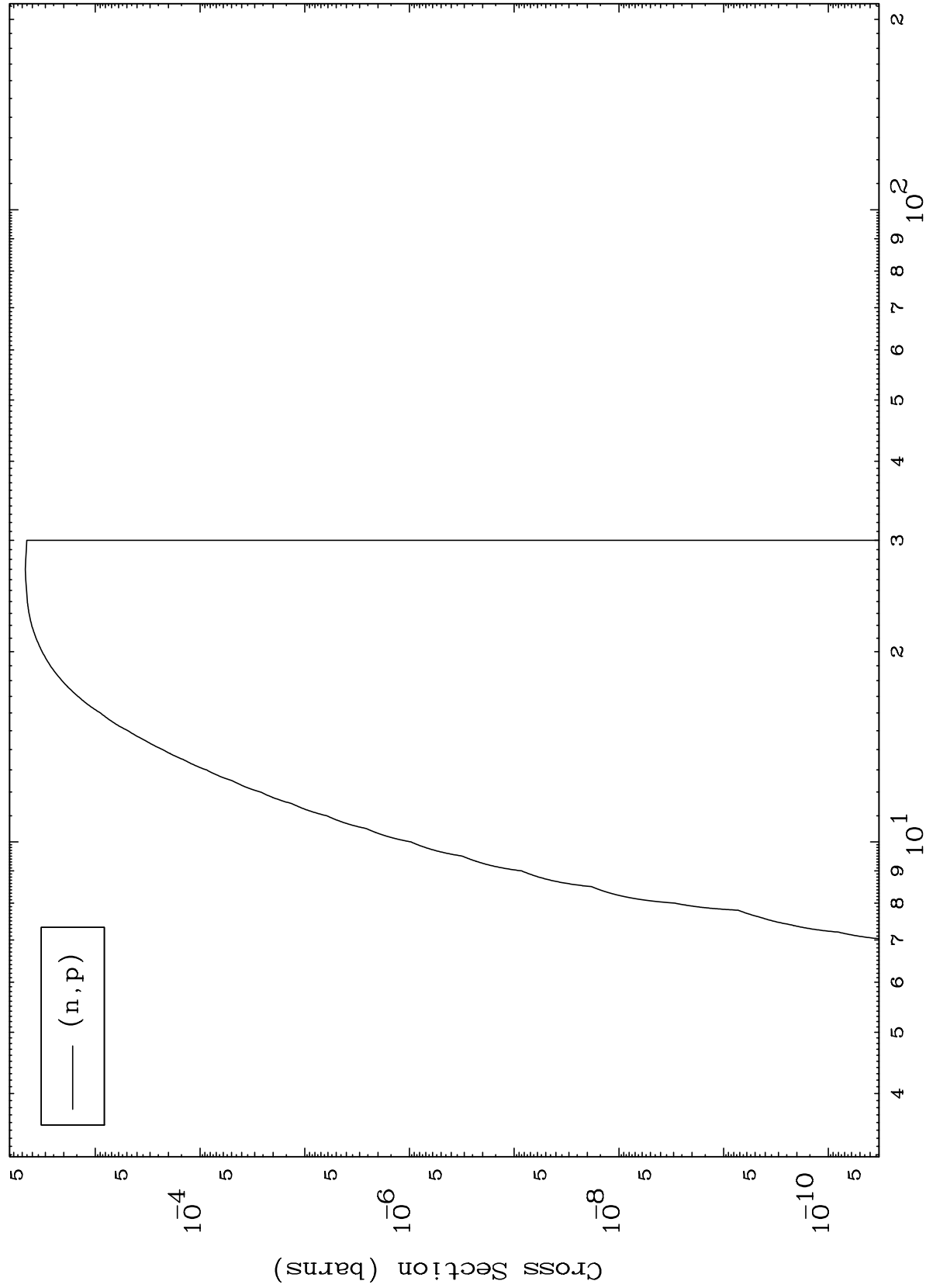




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85-At-225

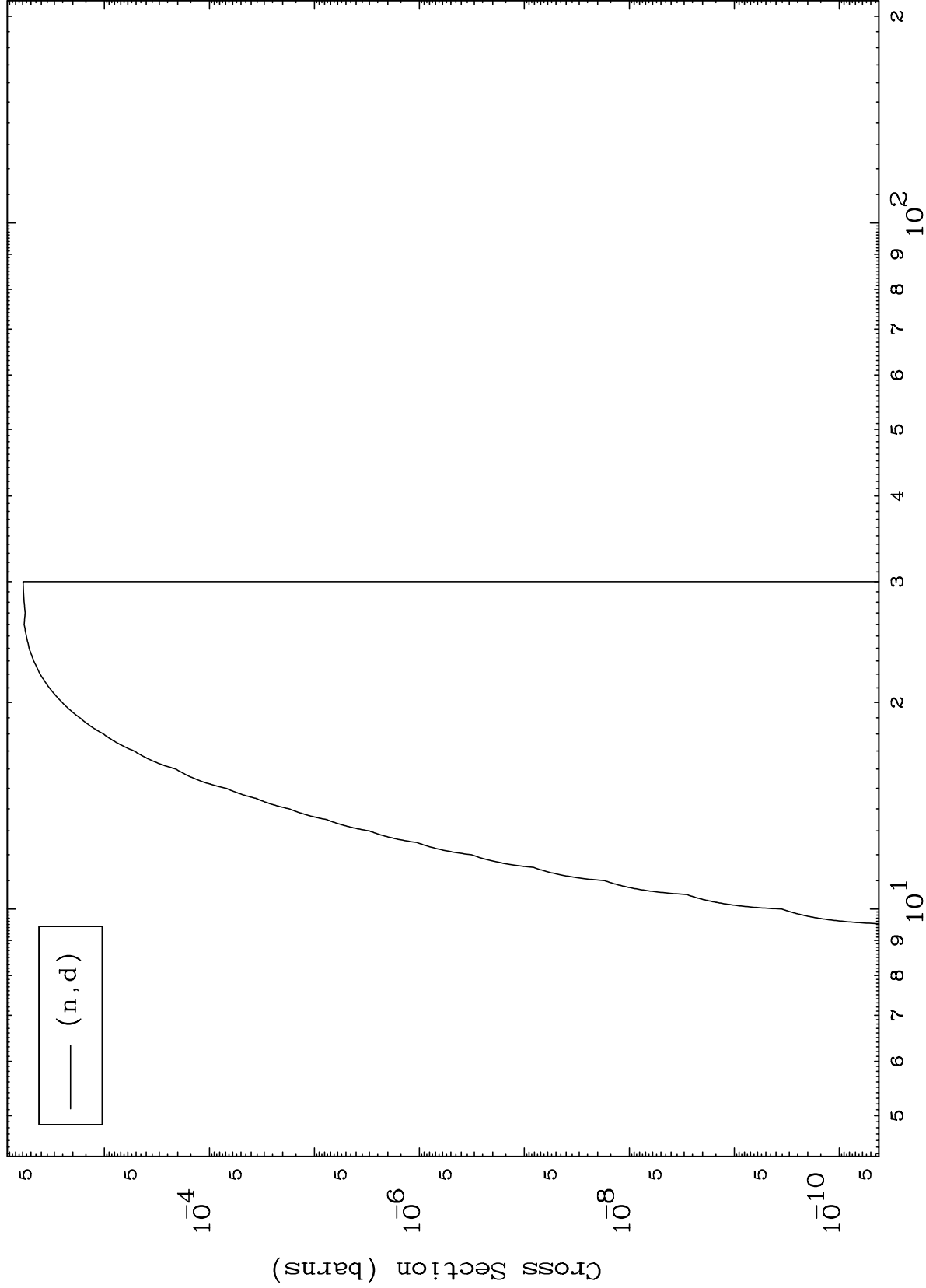
(n,p) Levels
293 Kelvin Cross Sections



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(n,d) Levels
293 Kelvin Cross Sections

85-At-225



10

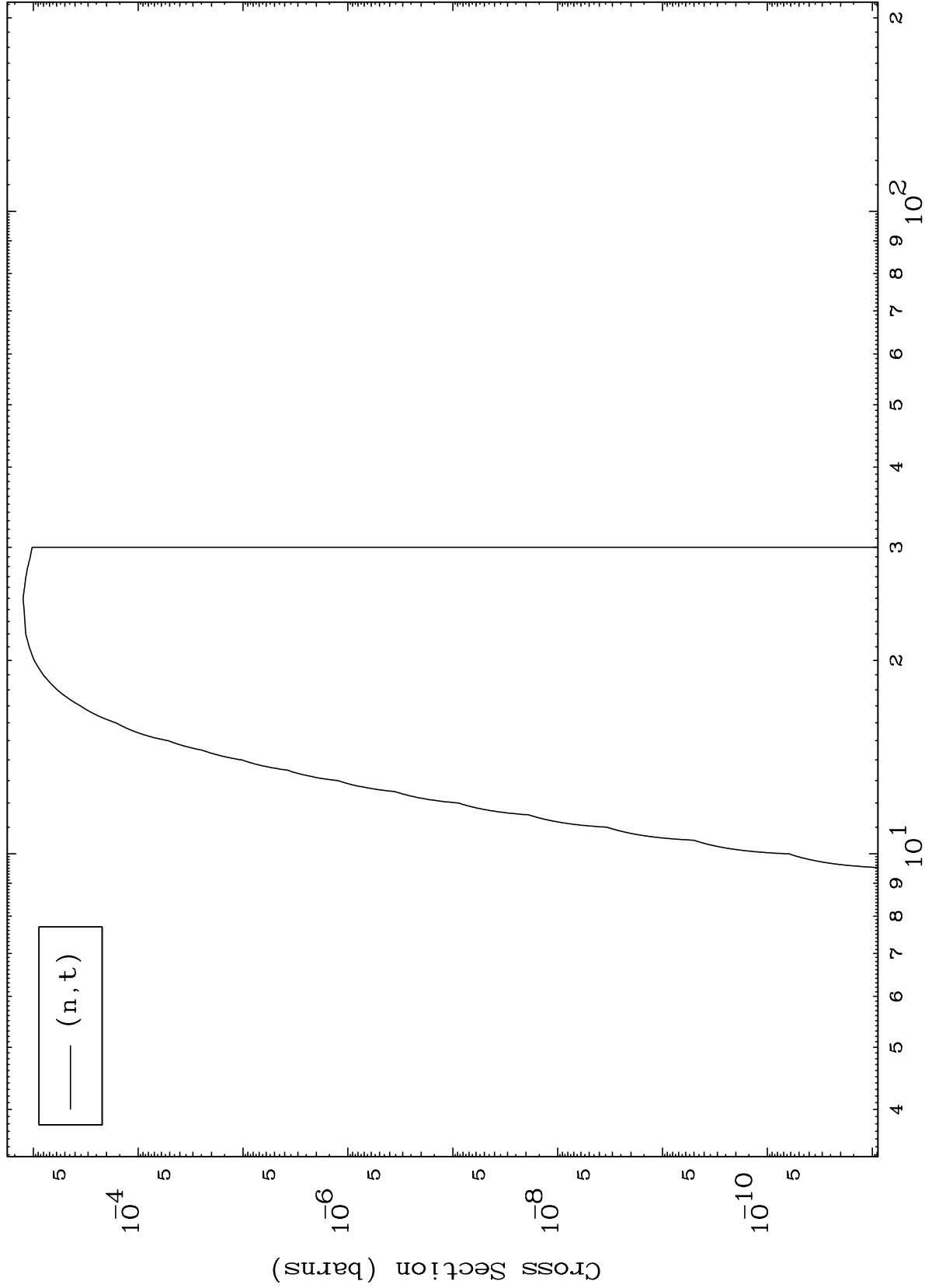
Incident Energy (MeV)

85-At-225

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85-At-225

(n,t) Levels
293 Kelvin Cross Sections



11

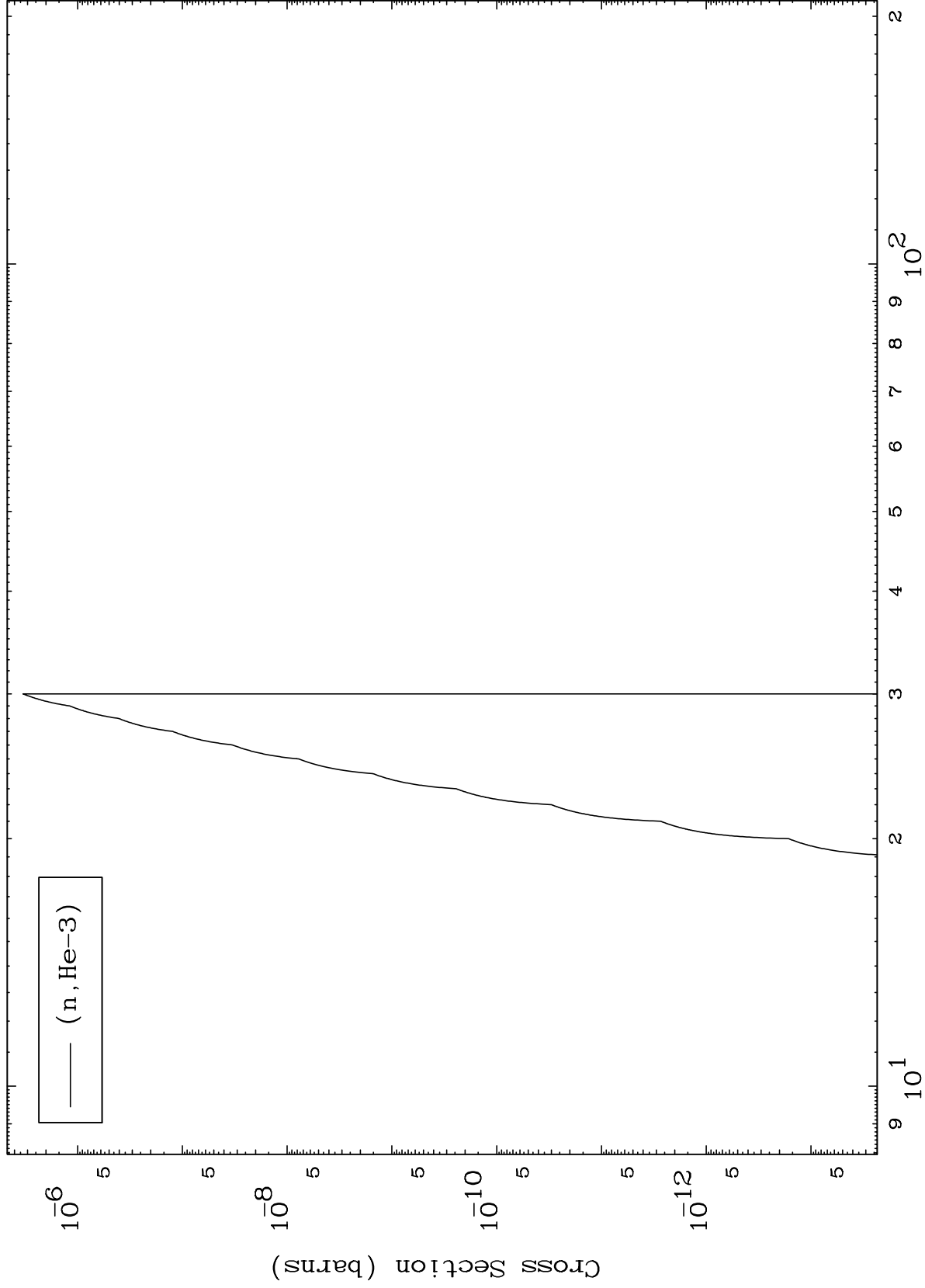
Incident Energy (MeV)

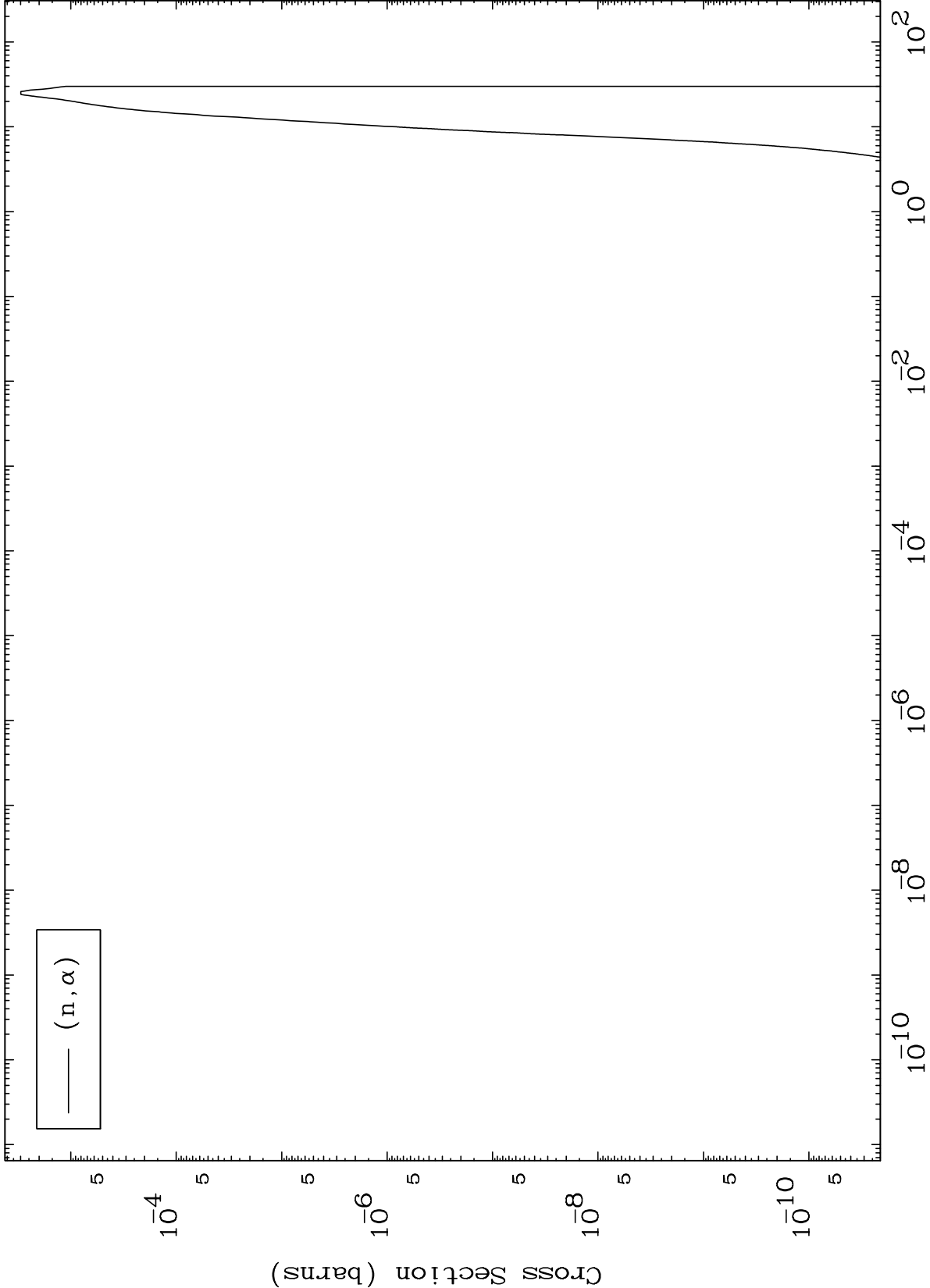
85-At-225

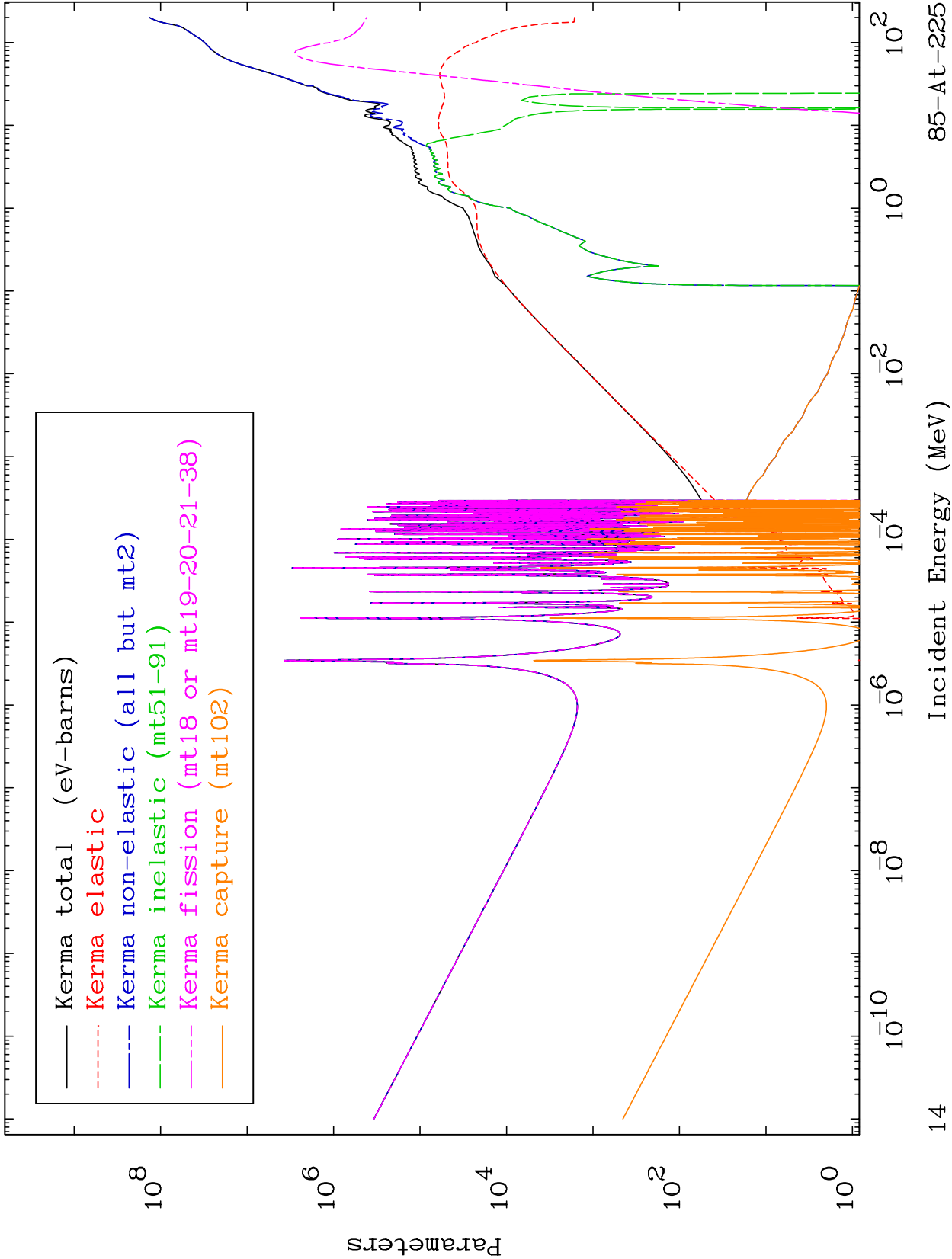
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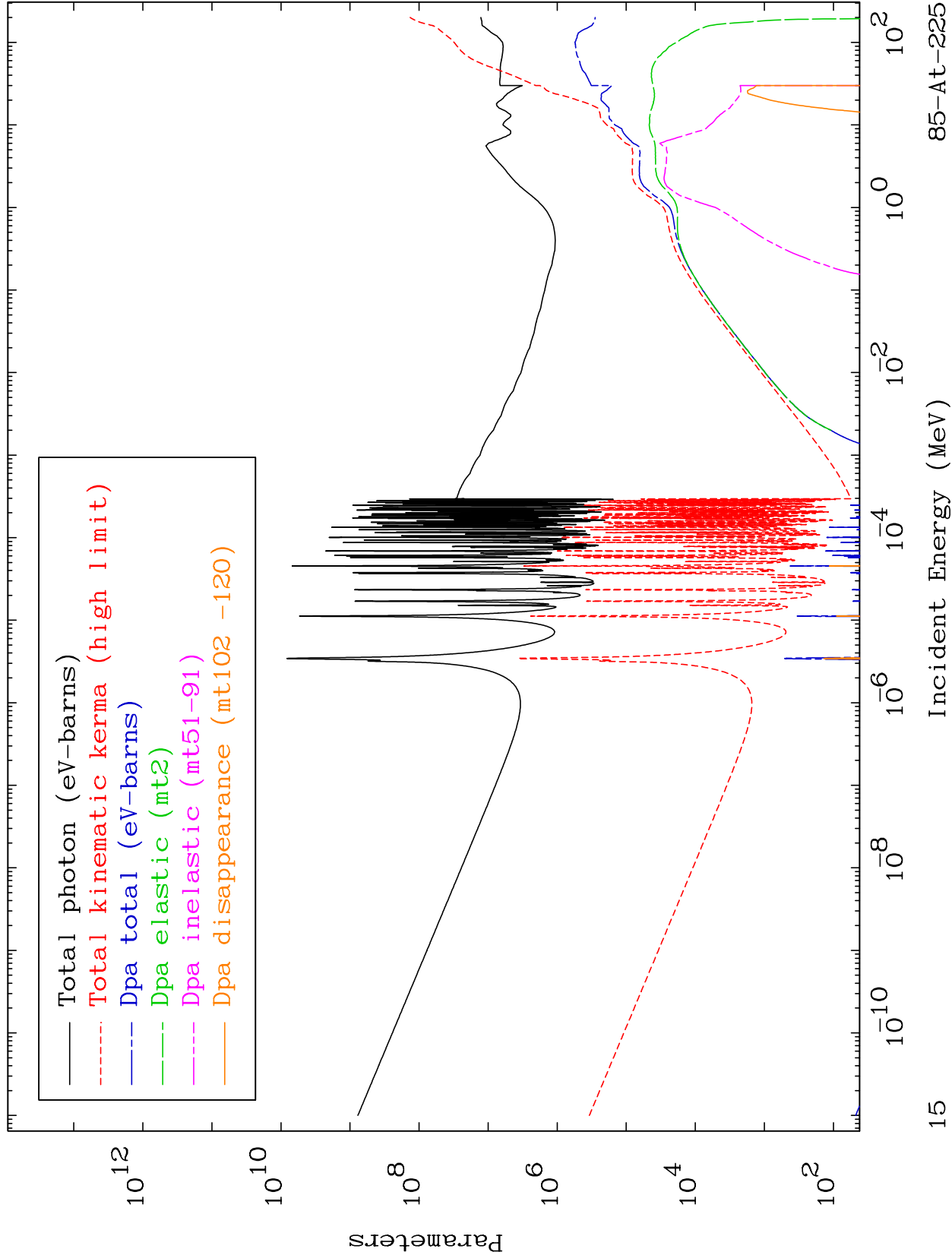
(n,He3) Levels
293 Kelvin Cross Sections

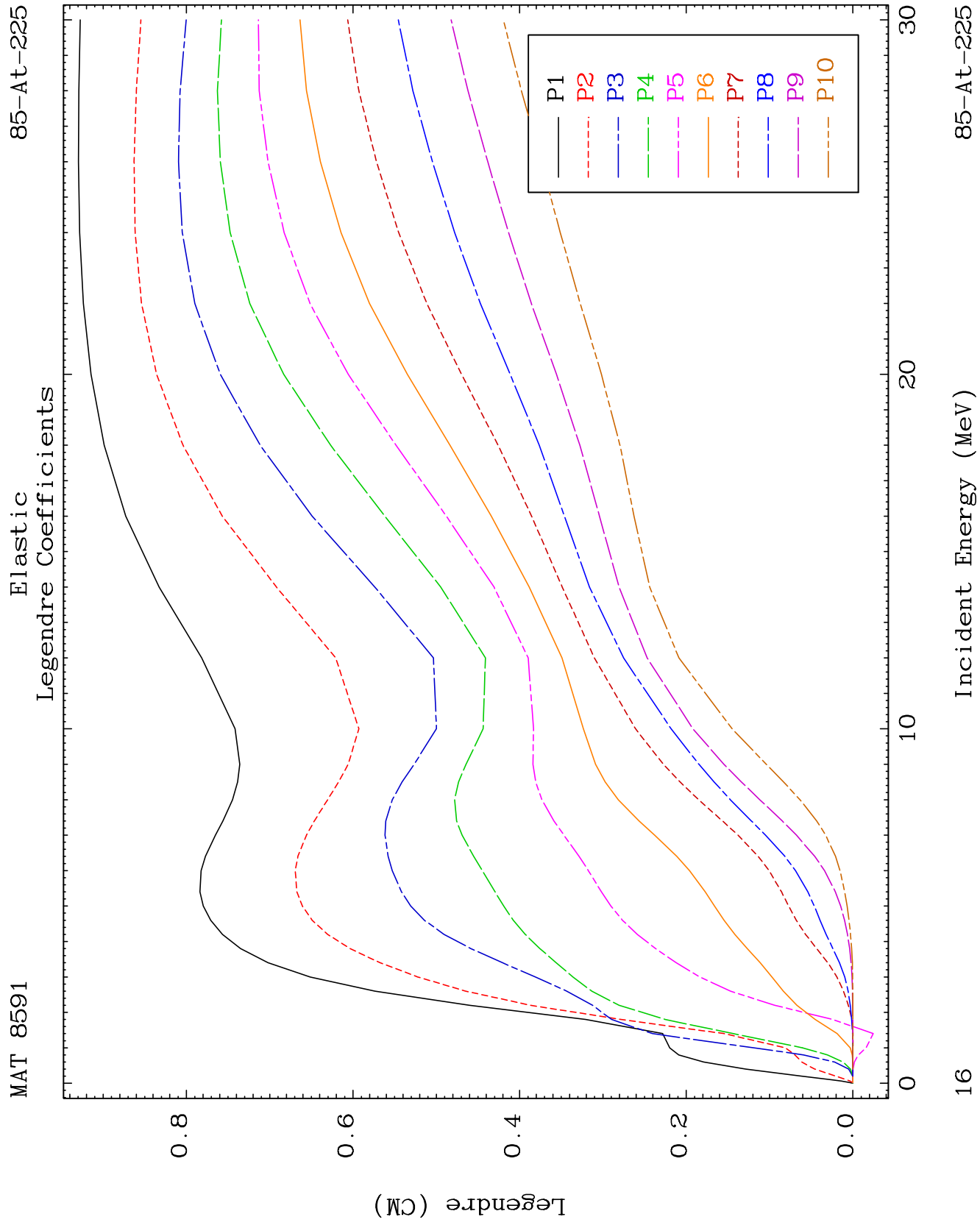
85-At-225

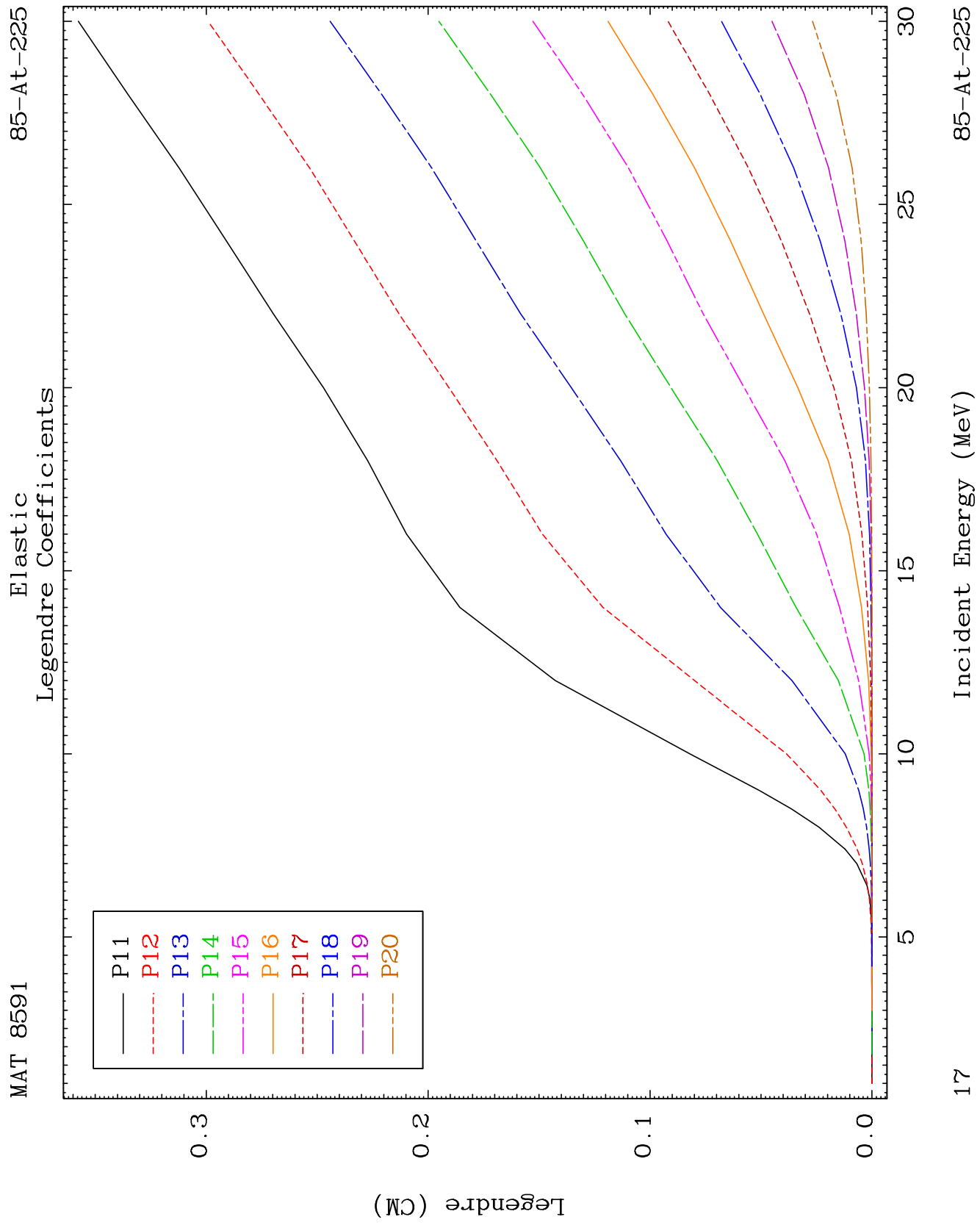


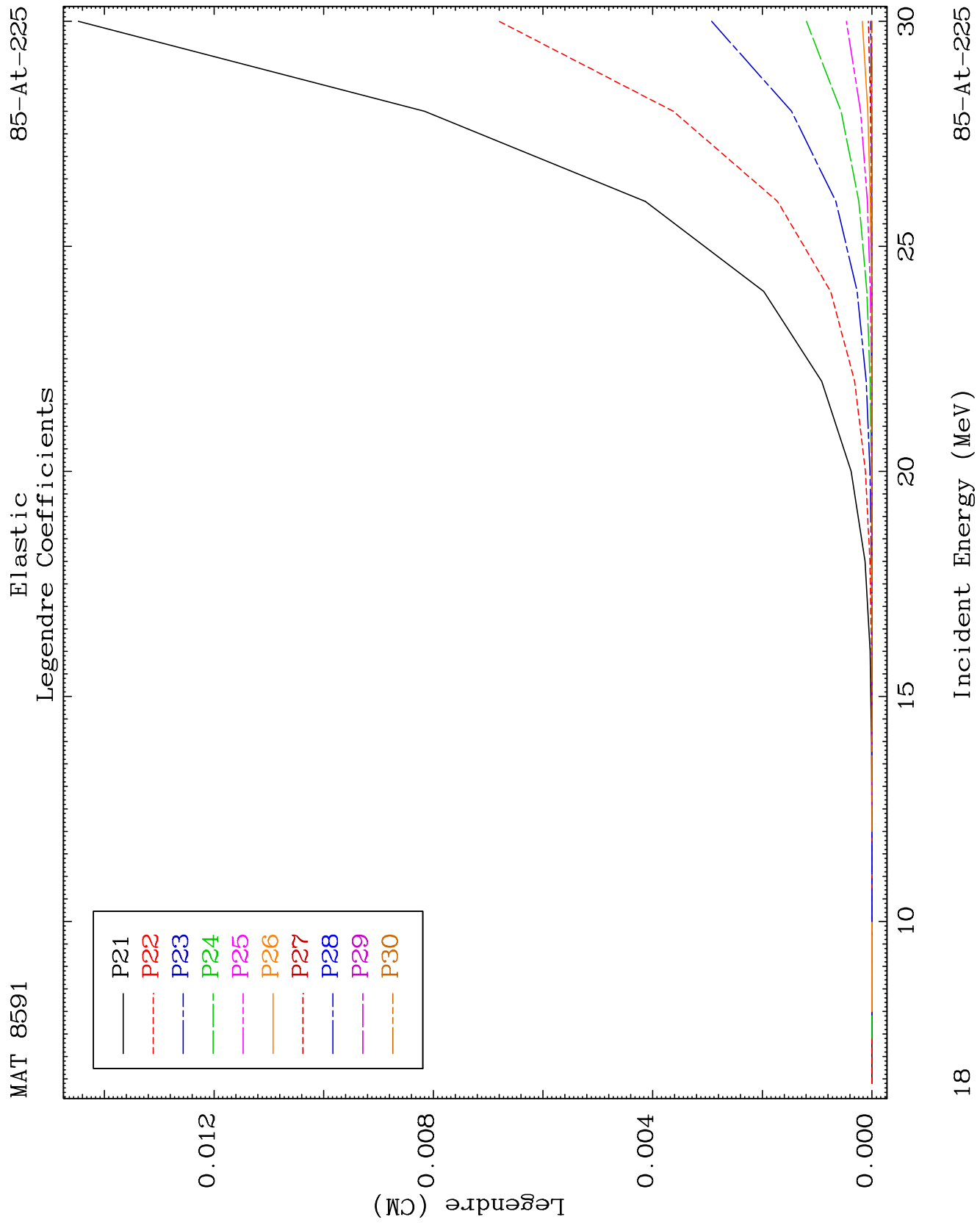


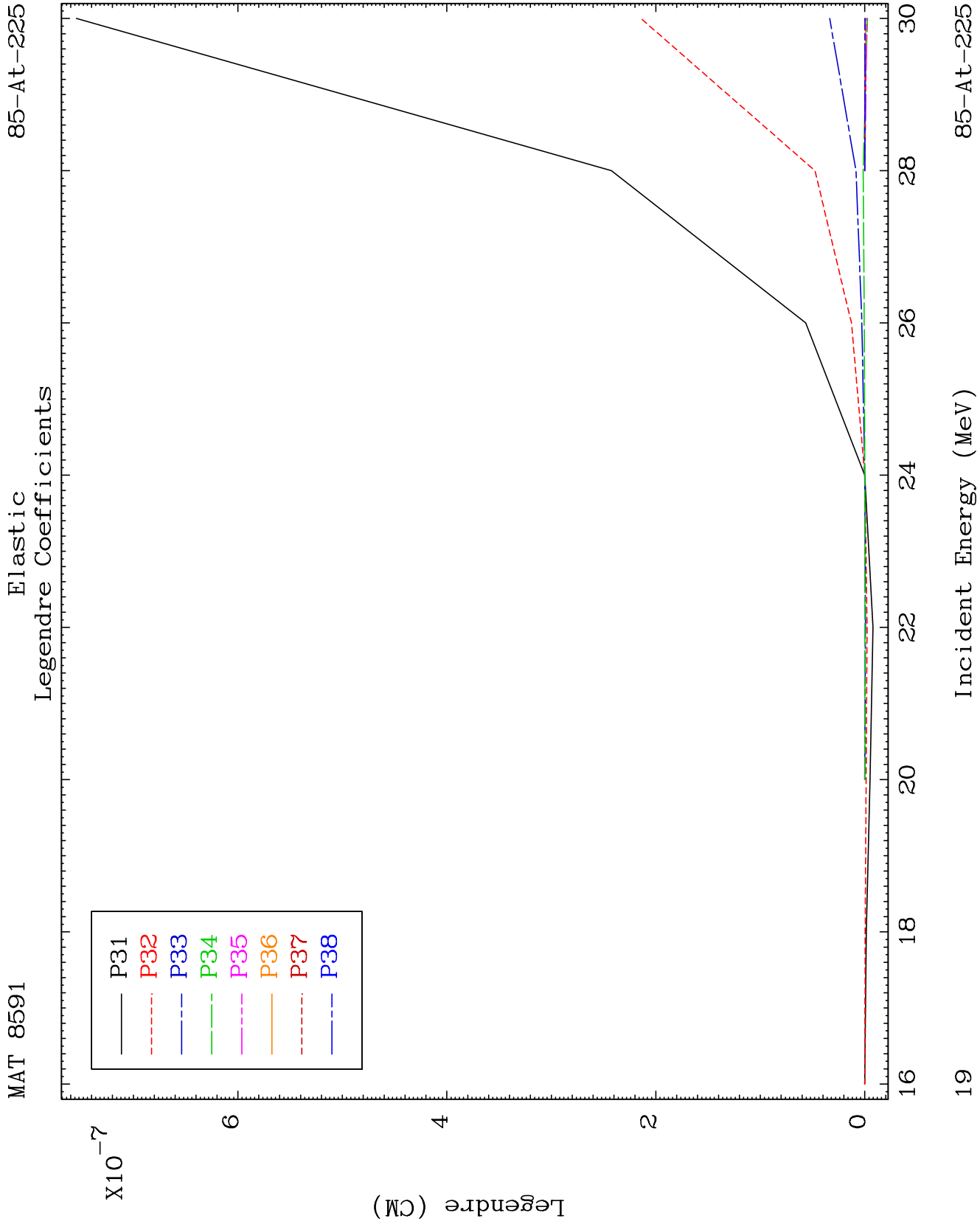








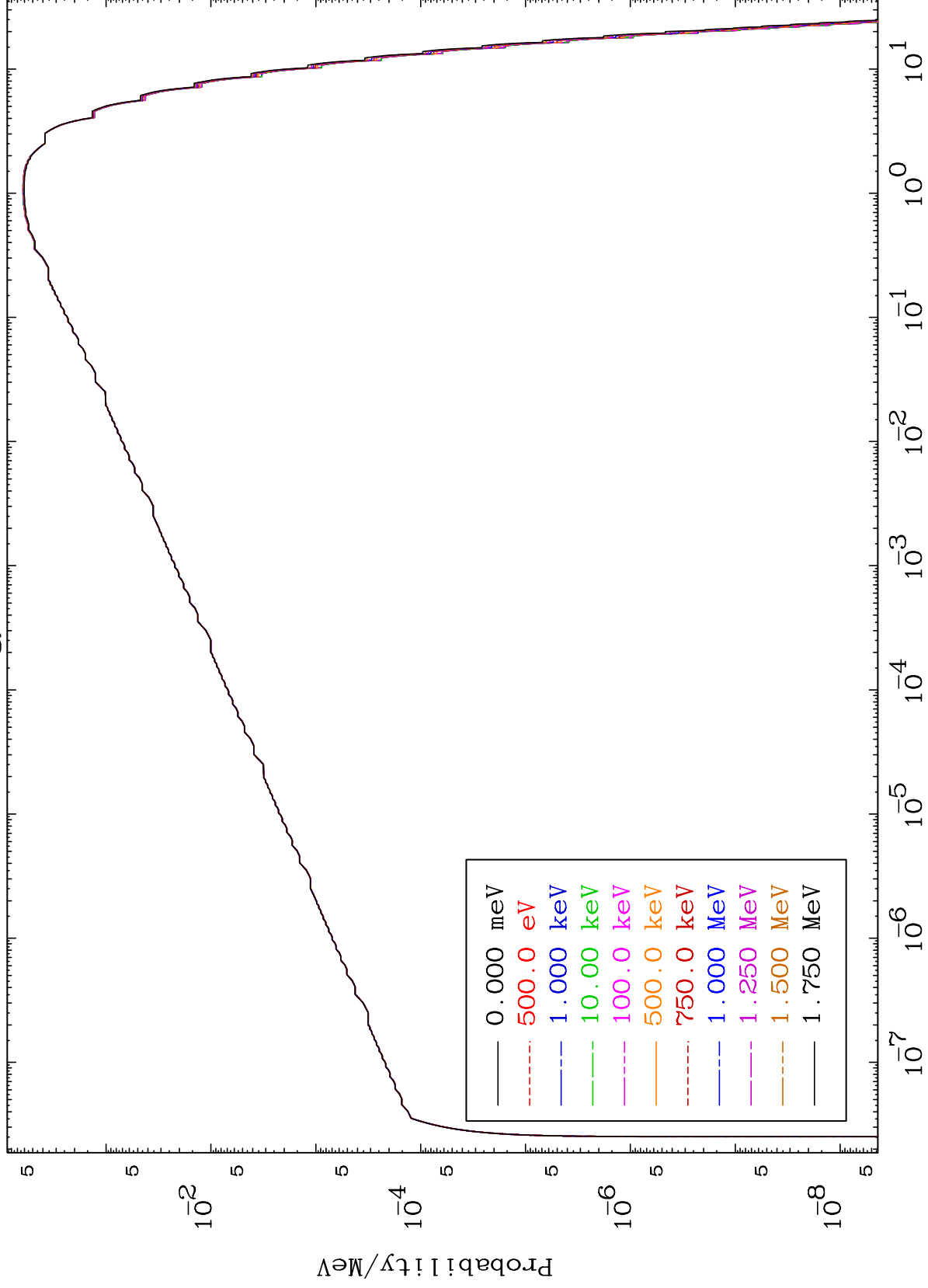


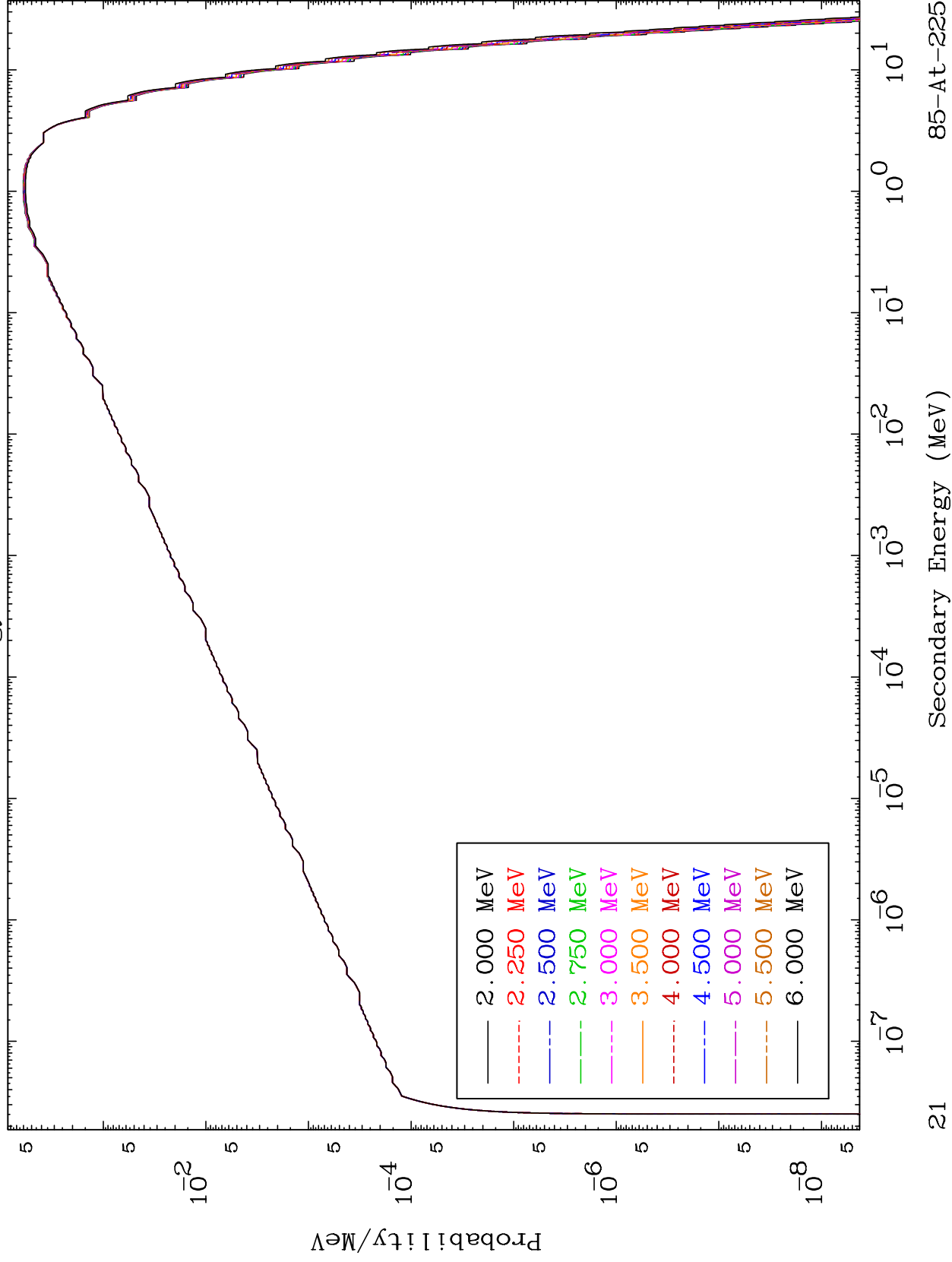


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Fission
Energy Distribution

85-At-225

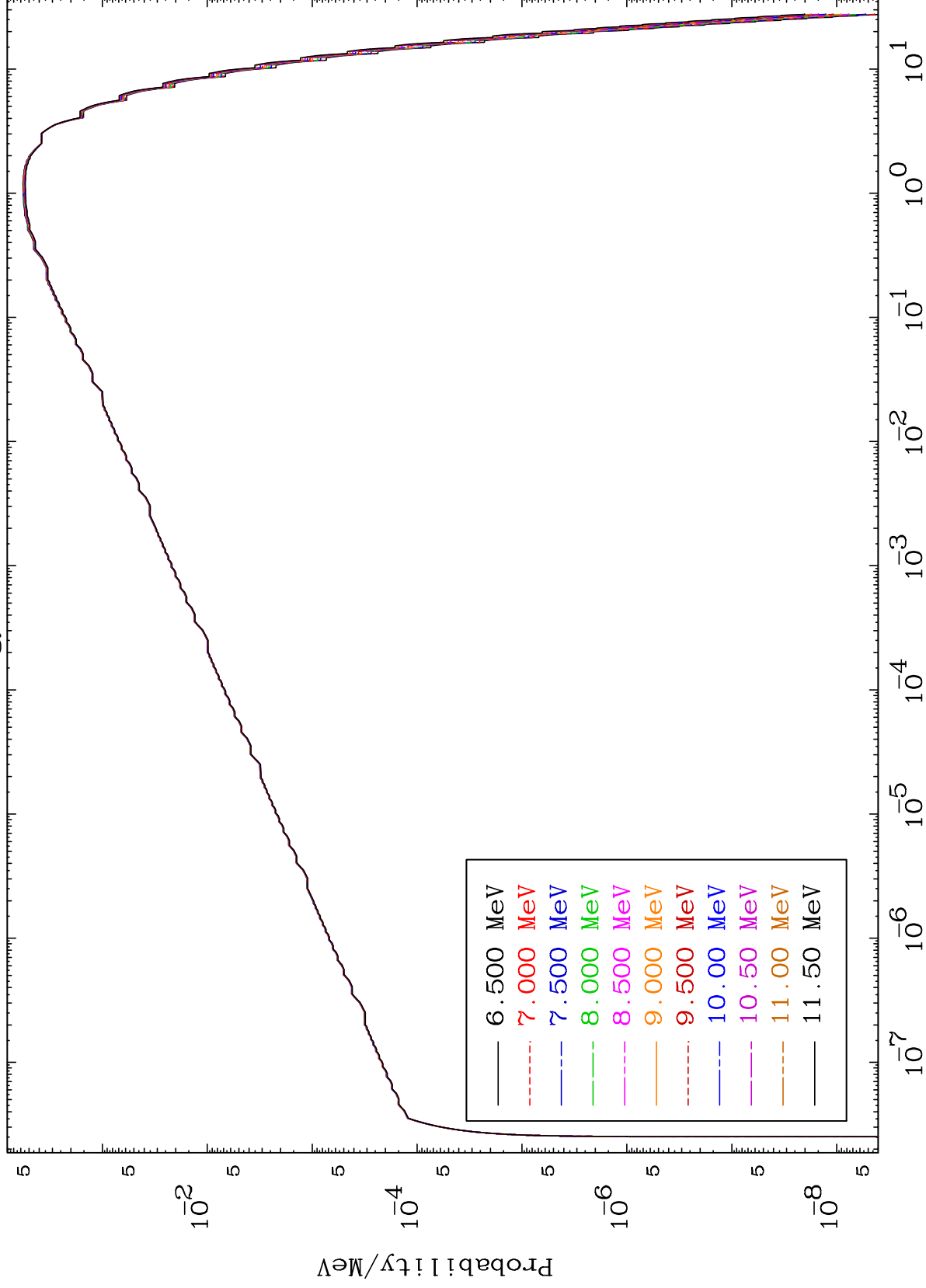


Fission
Energy Distribution

MAT 8591

Fission Energy Distribution

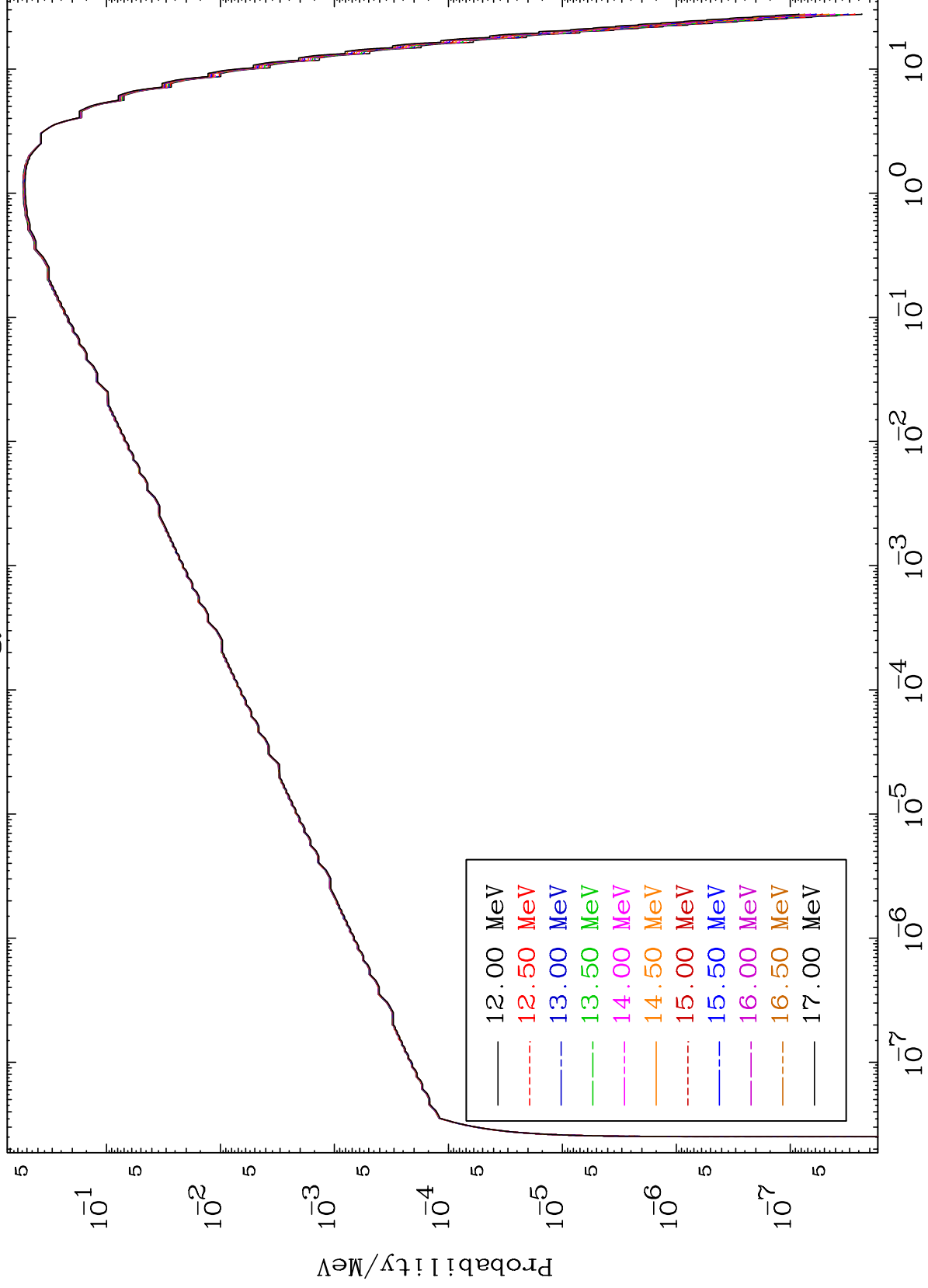
85-At-225



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Fission Energy Distribution

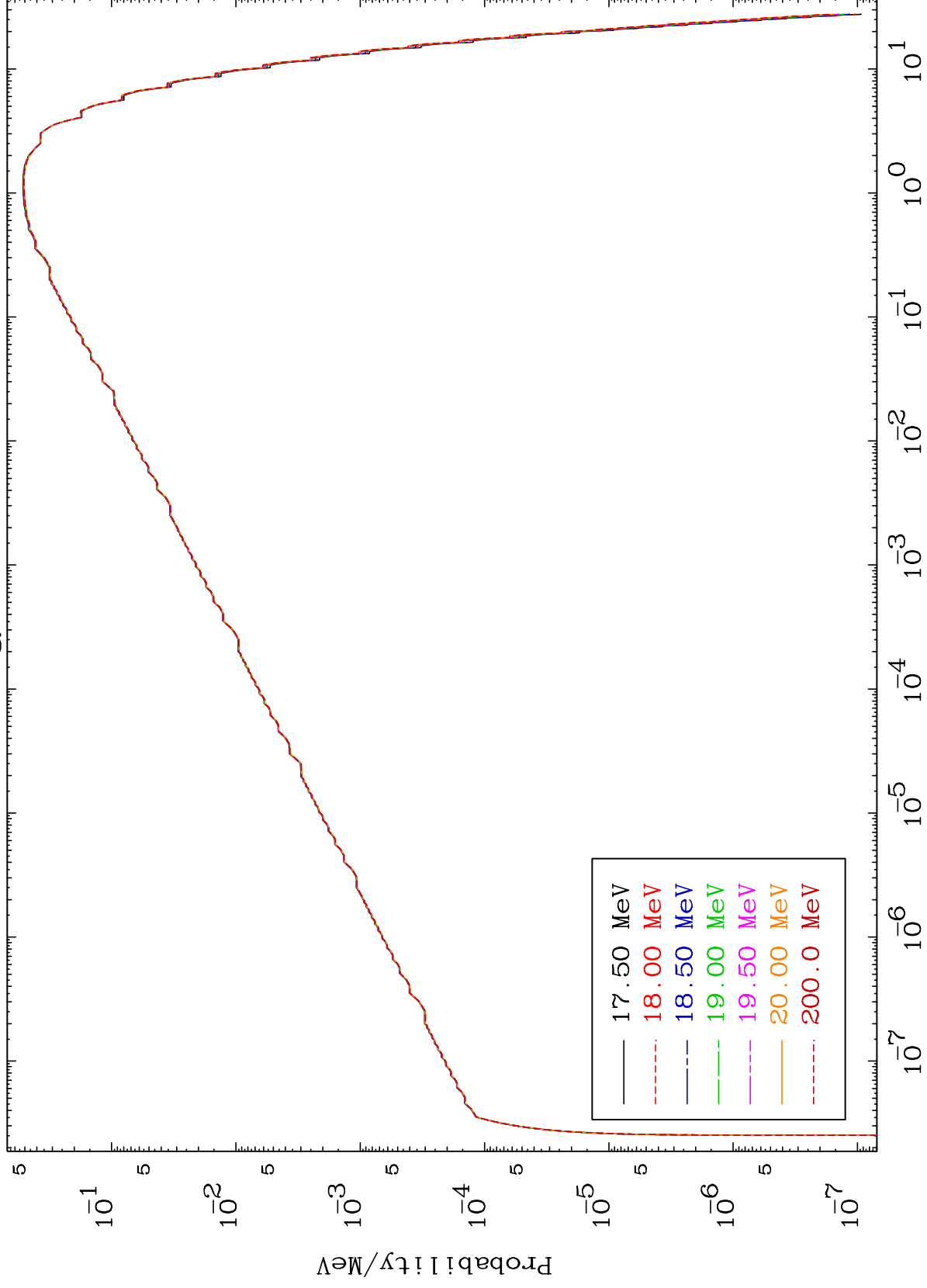
85-At-225



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Fission
Energy Distribution

85-At-225



Secondary Energy (MeV)

85-At-225