

Program EVALPLOT  
(Version 2017-1)

by

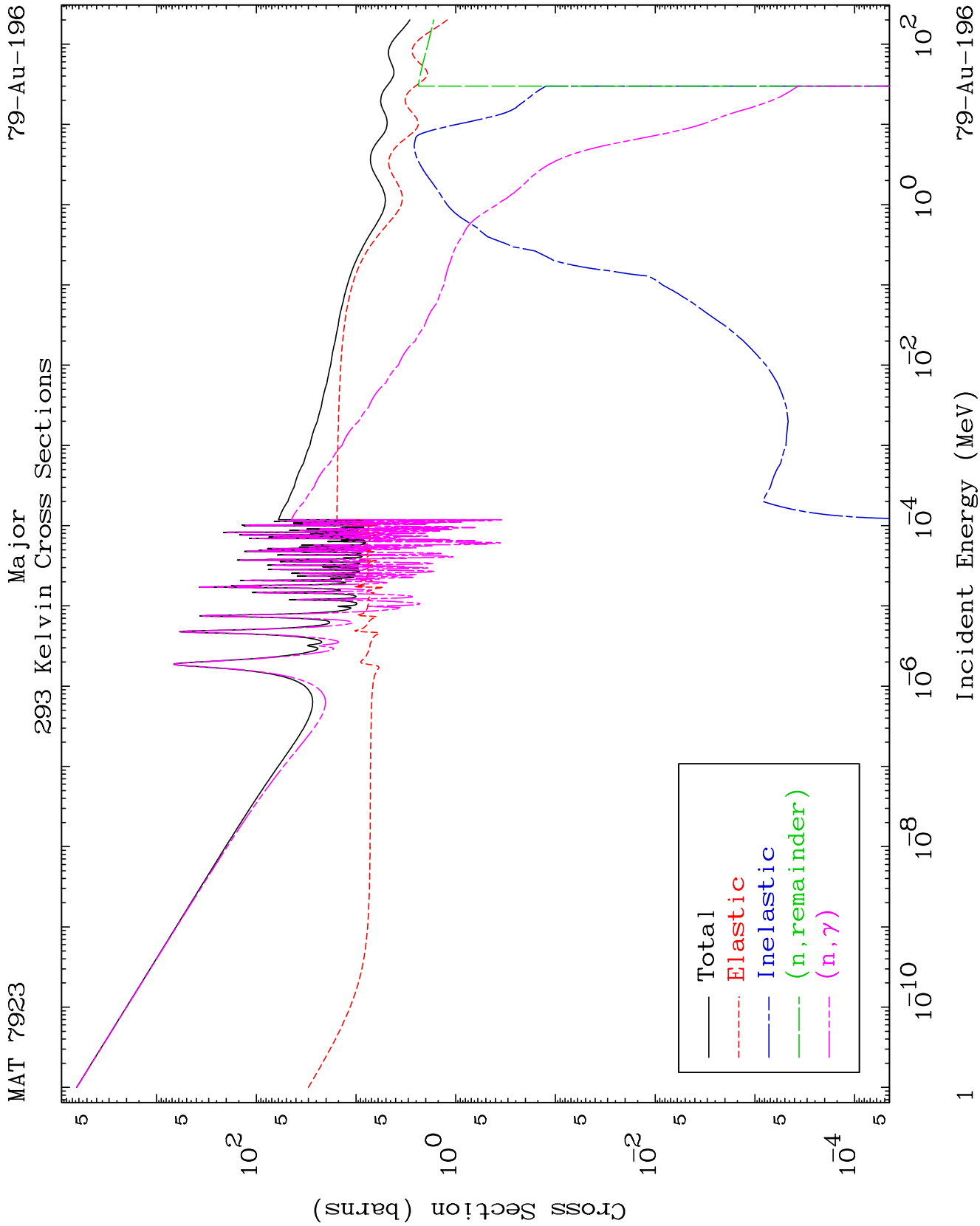
Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

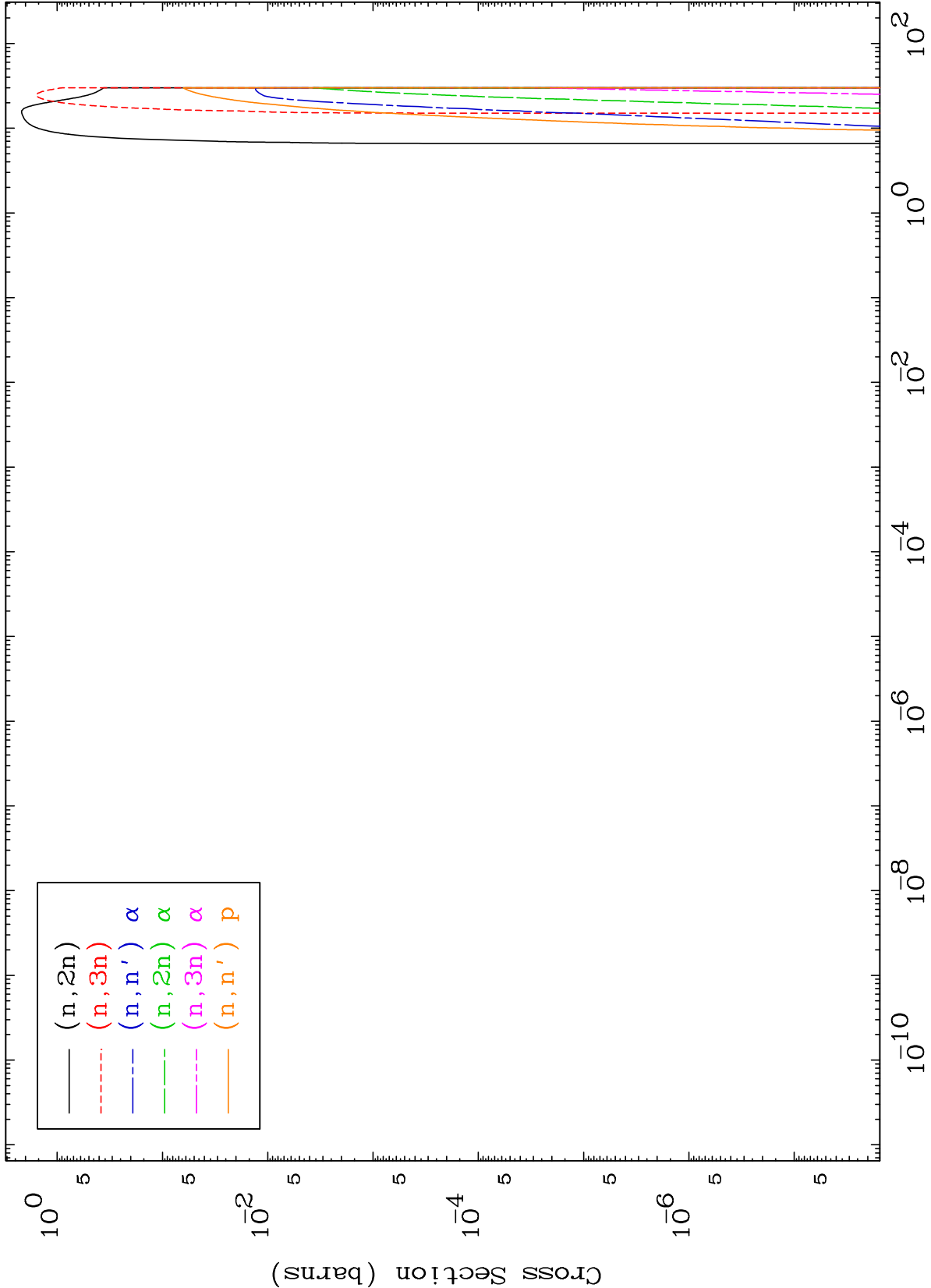
Press Mouse Button to Start

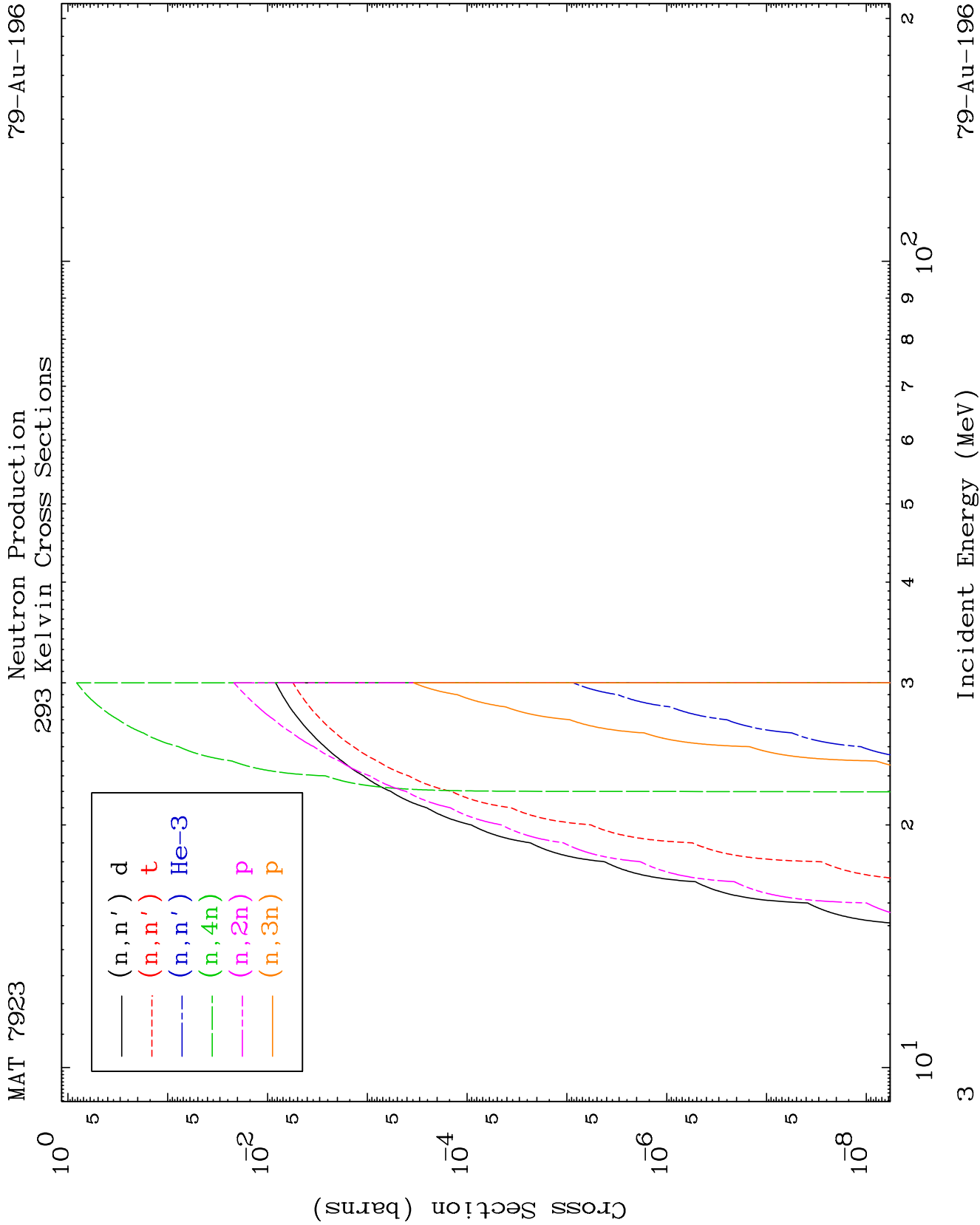


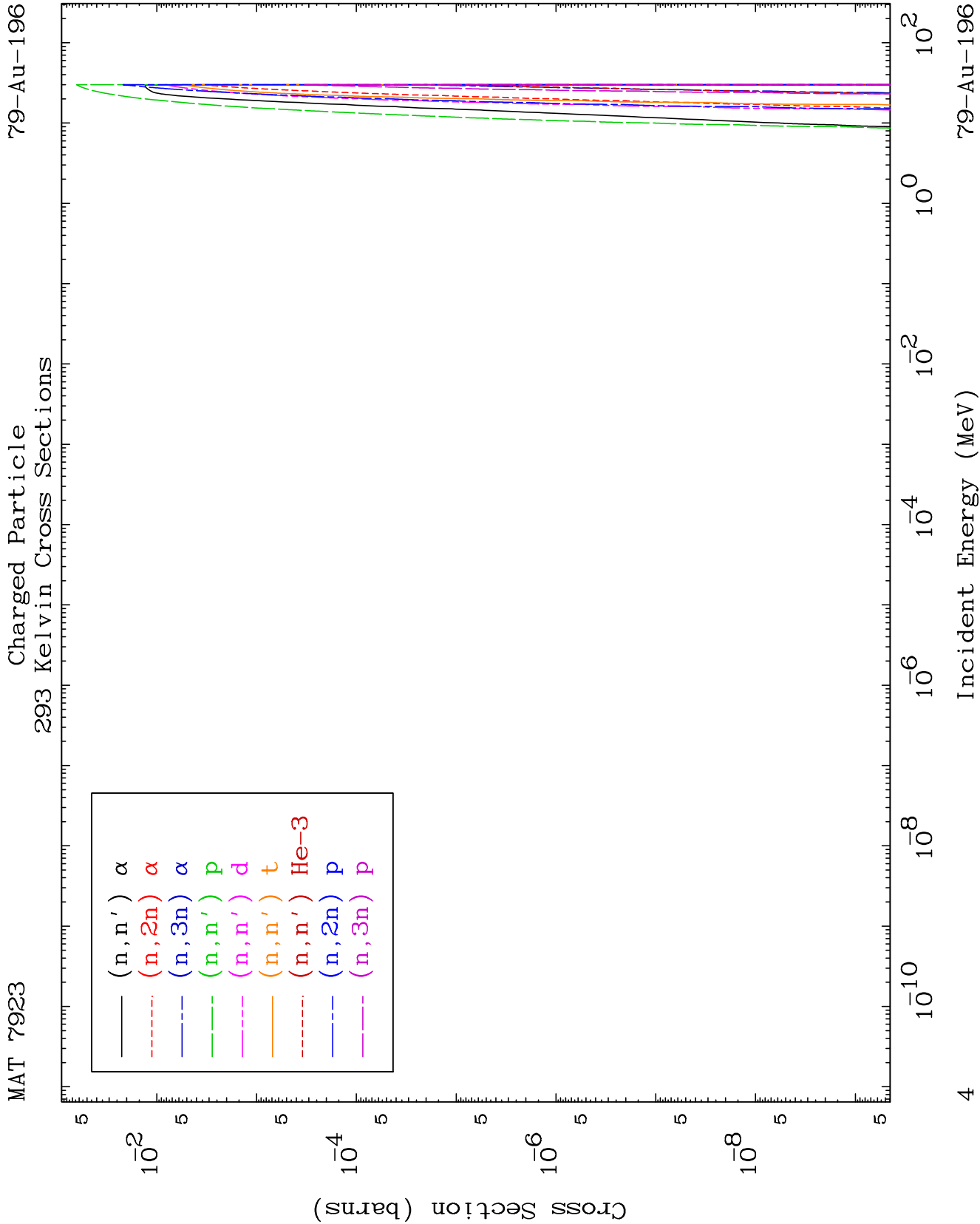
MAT 7923

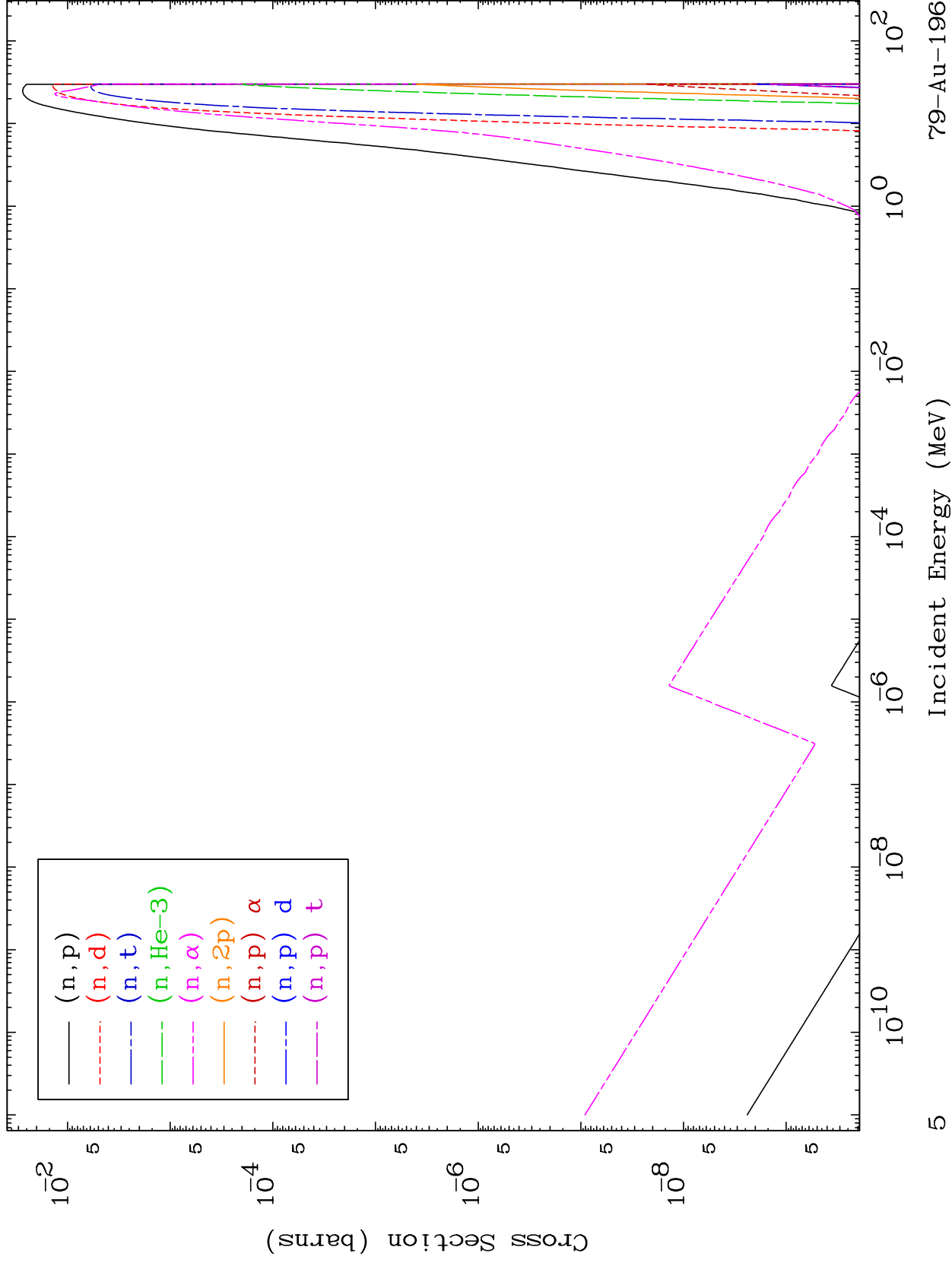
Neutron Production  
293 Kelvin Cross Sections

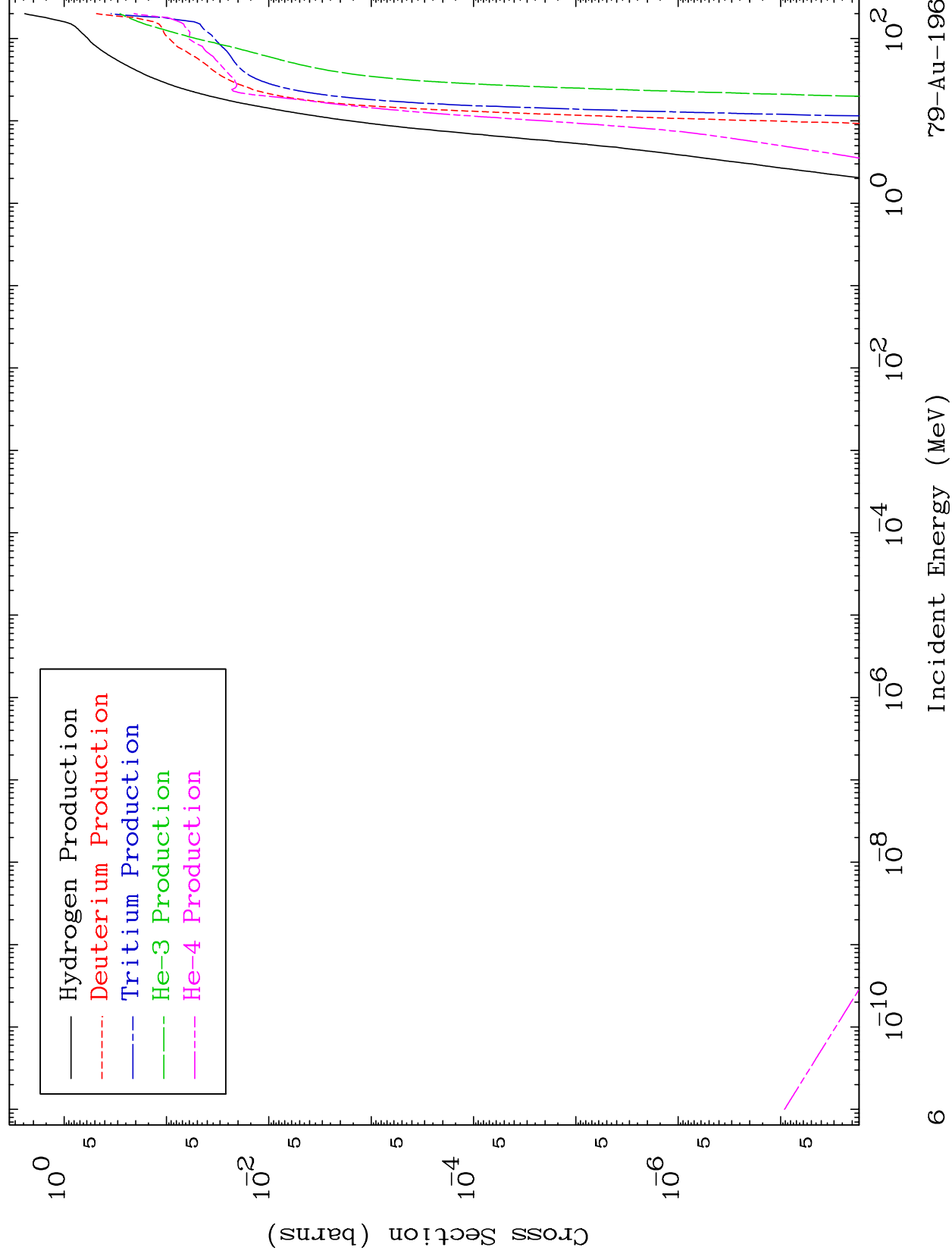
79-Au-196

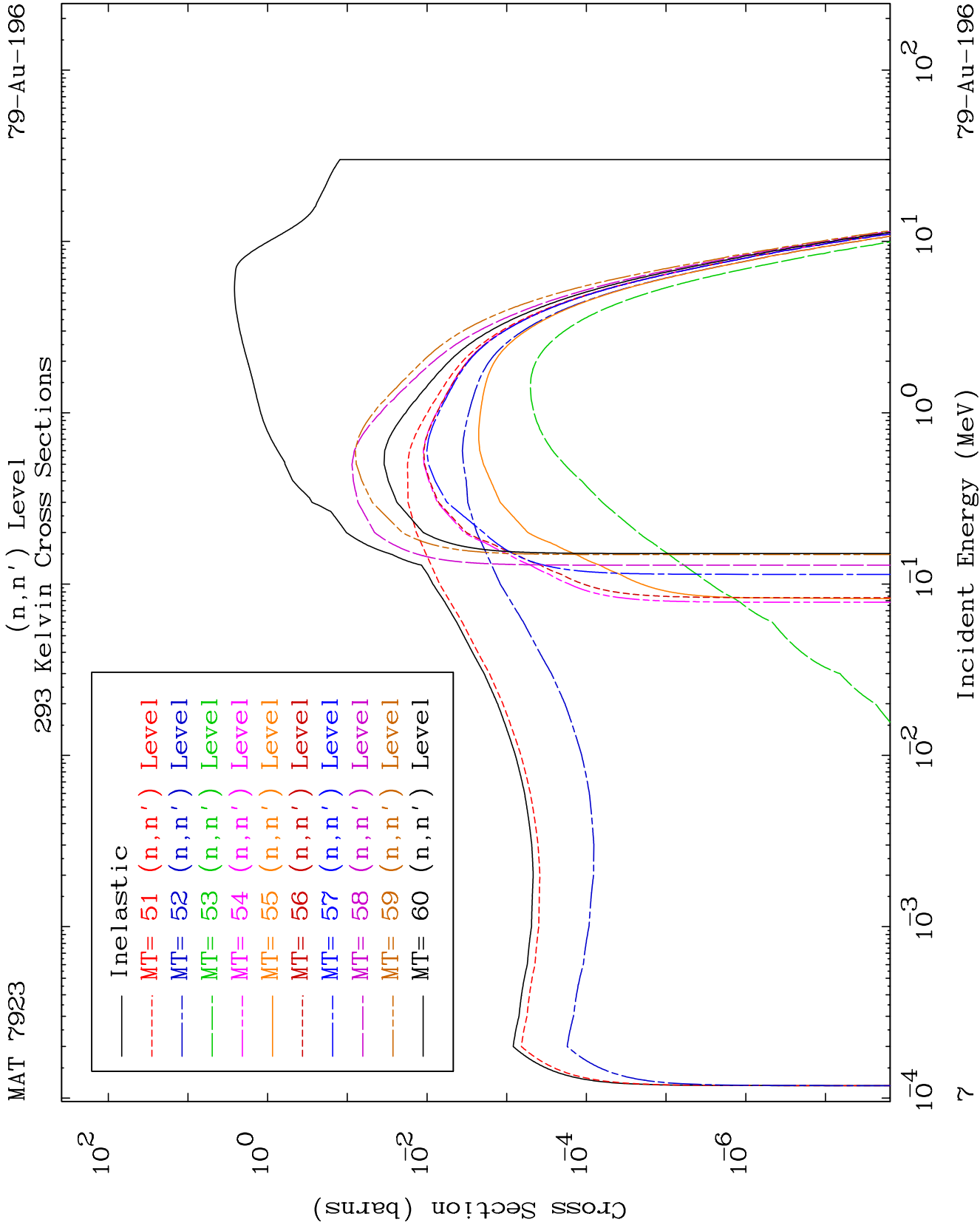


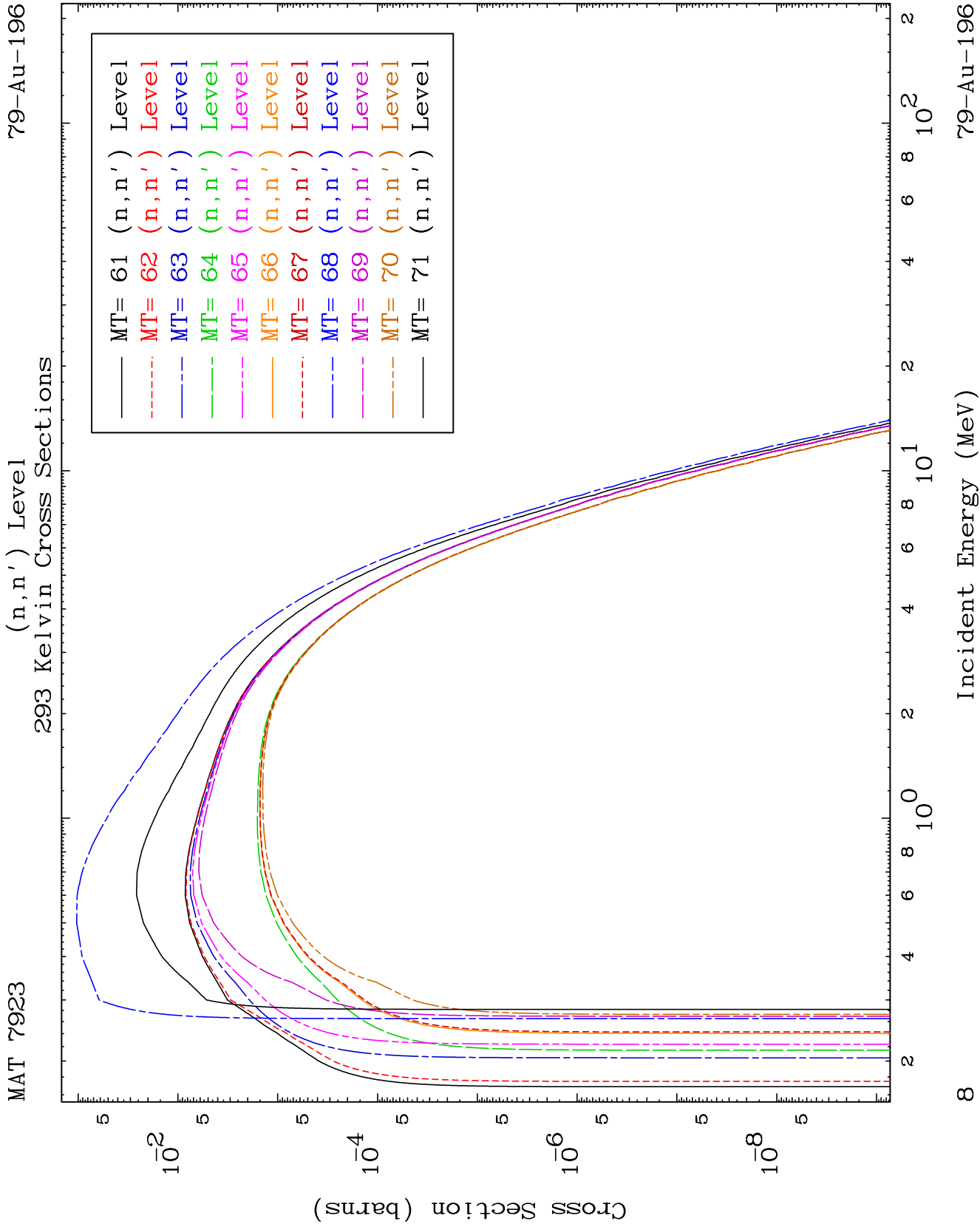


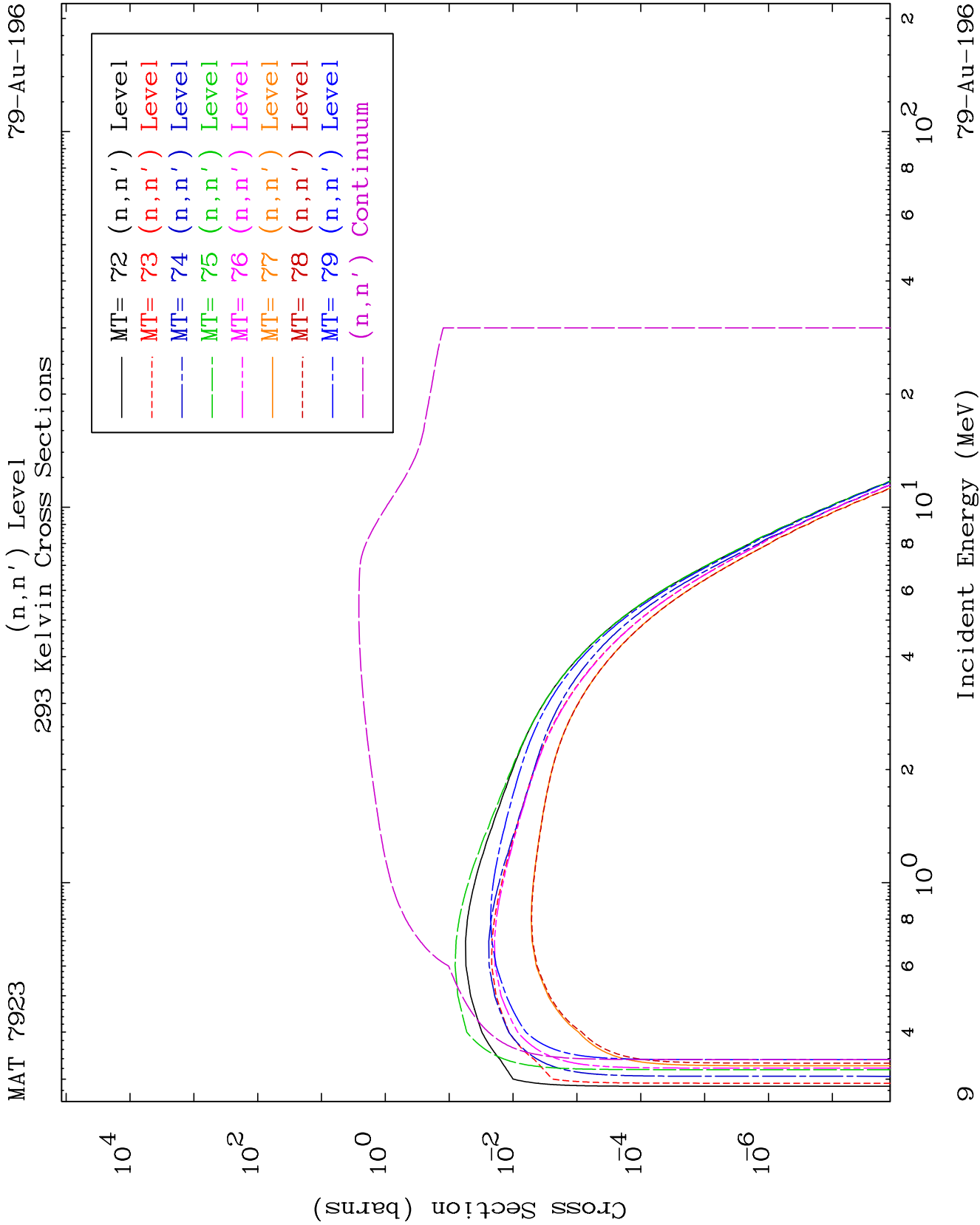








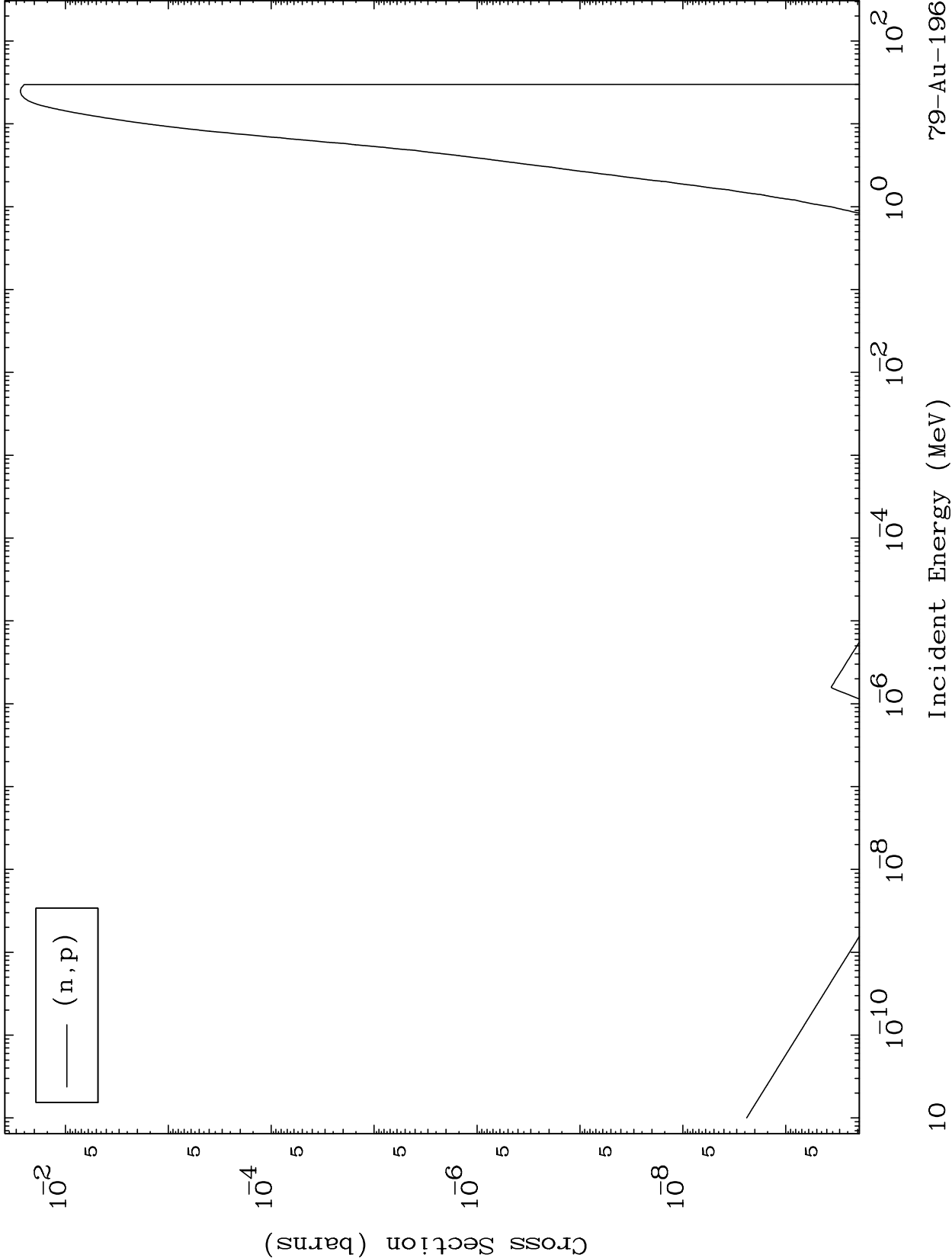




MAT 7923

(n,p) Levels  
293 Kelvin Cross Sections

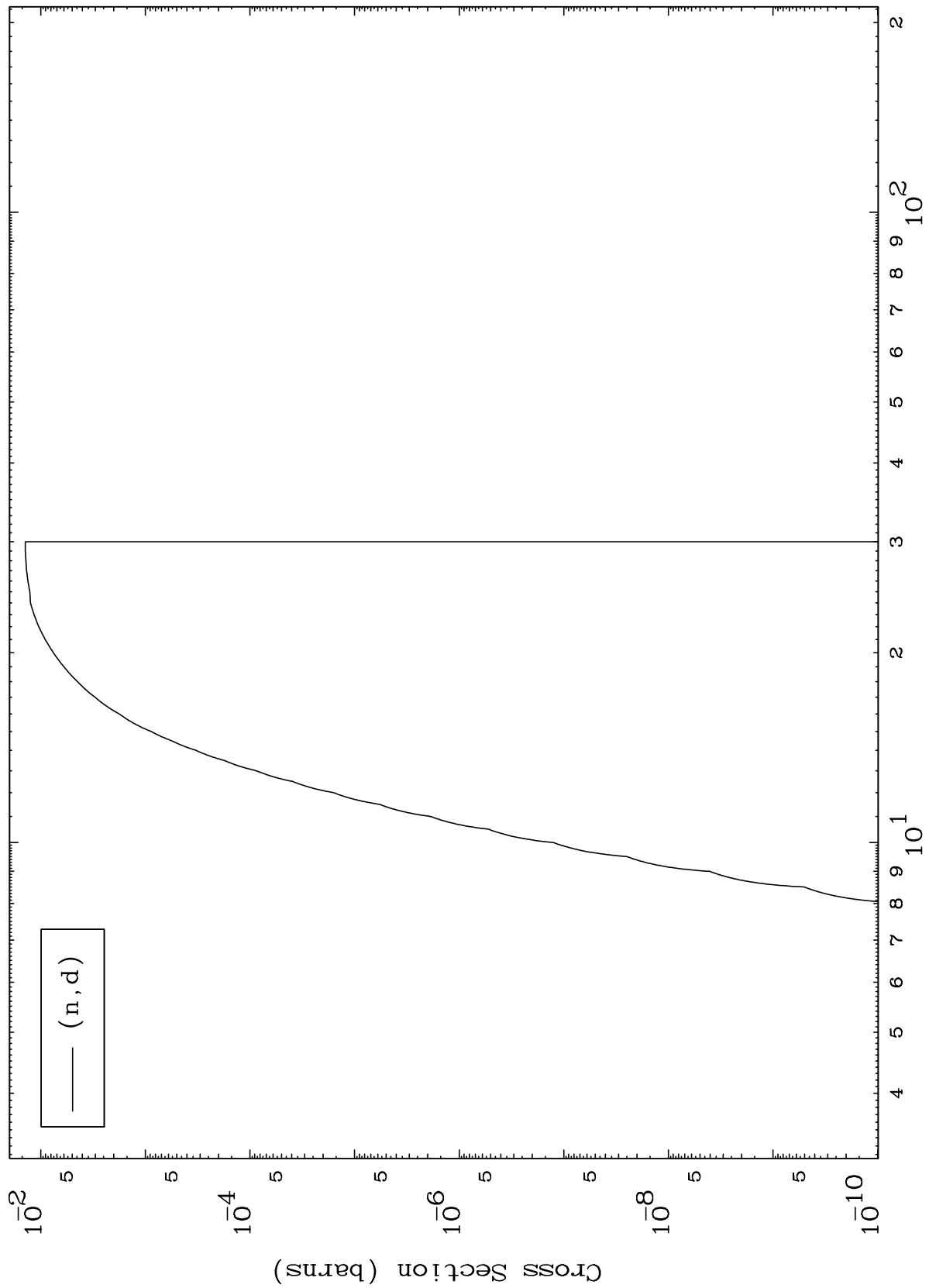
79-Au-196



MAT 7923

79-Au-196

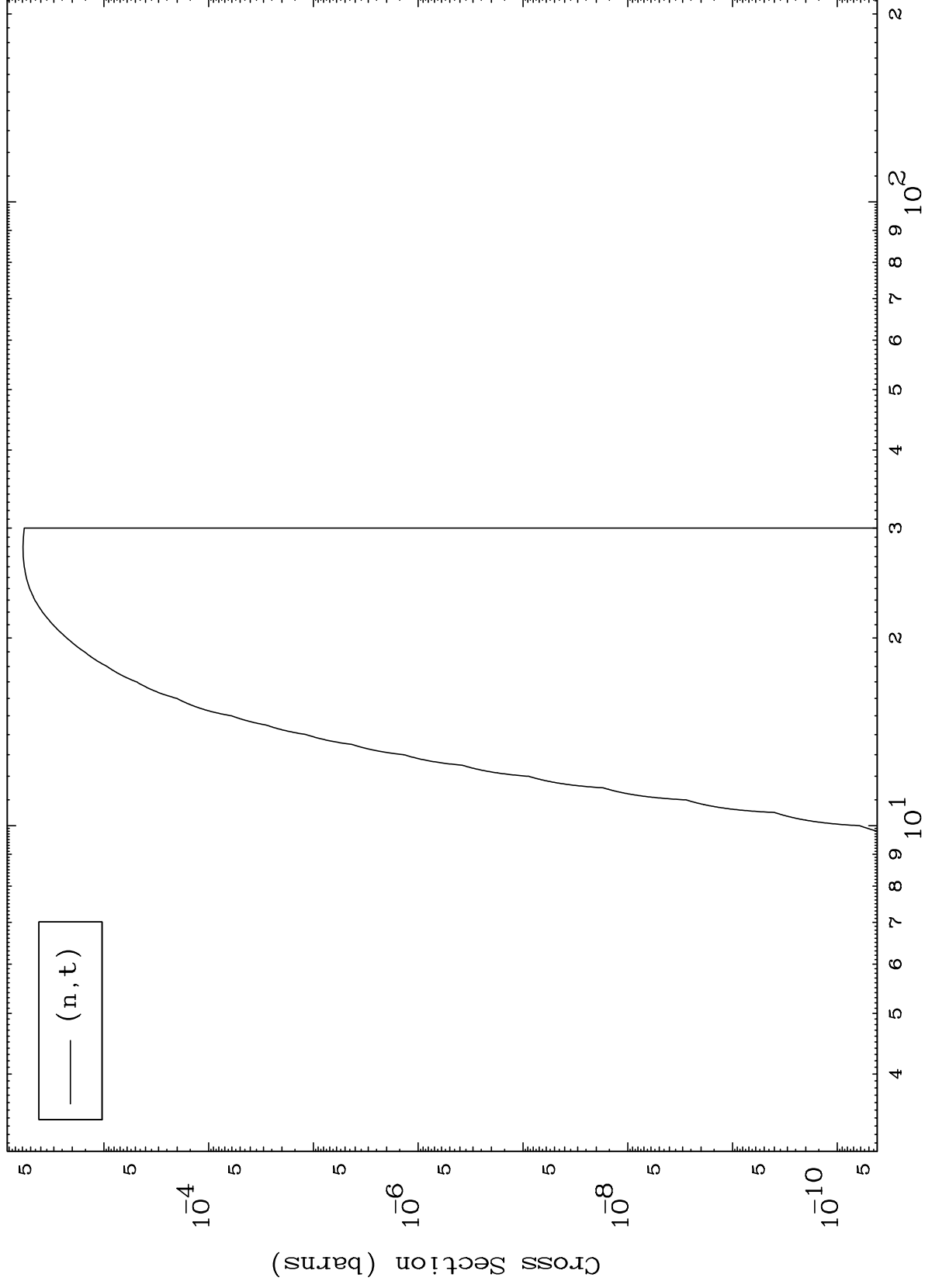
(n,d) Levels  
293 Kelvin Cross Sections



MAT 7923

79-Au-196

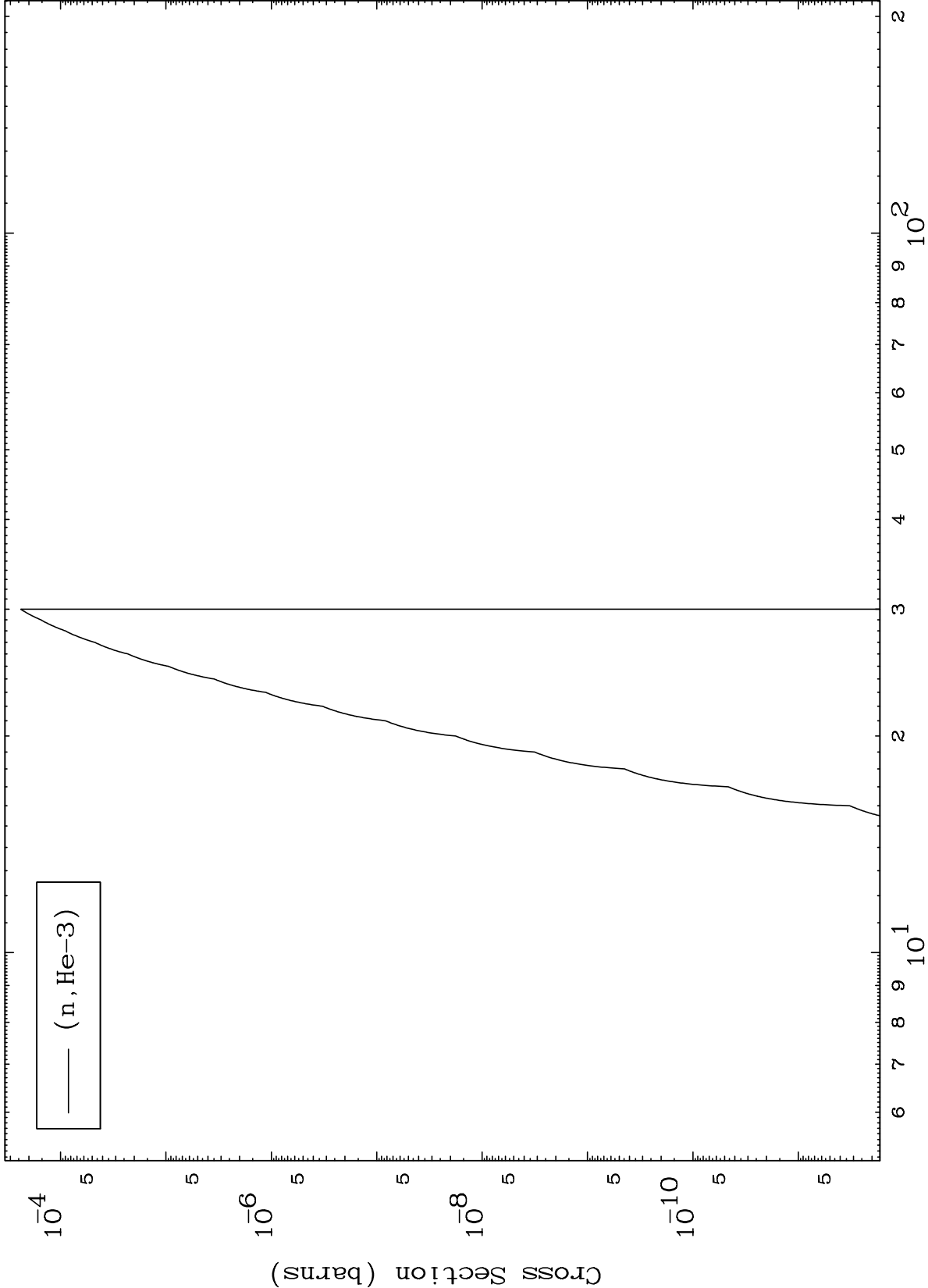
(n,t) Levels  
293 Kelvin Cross Sections

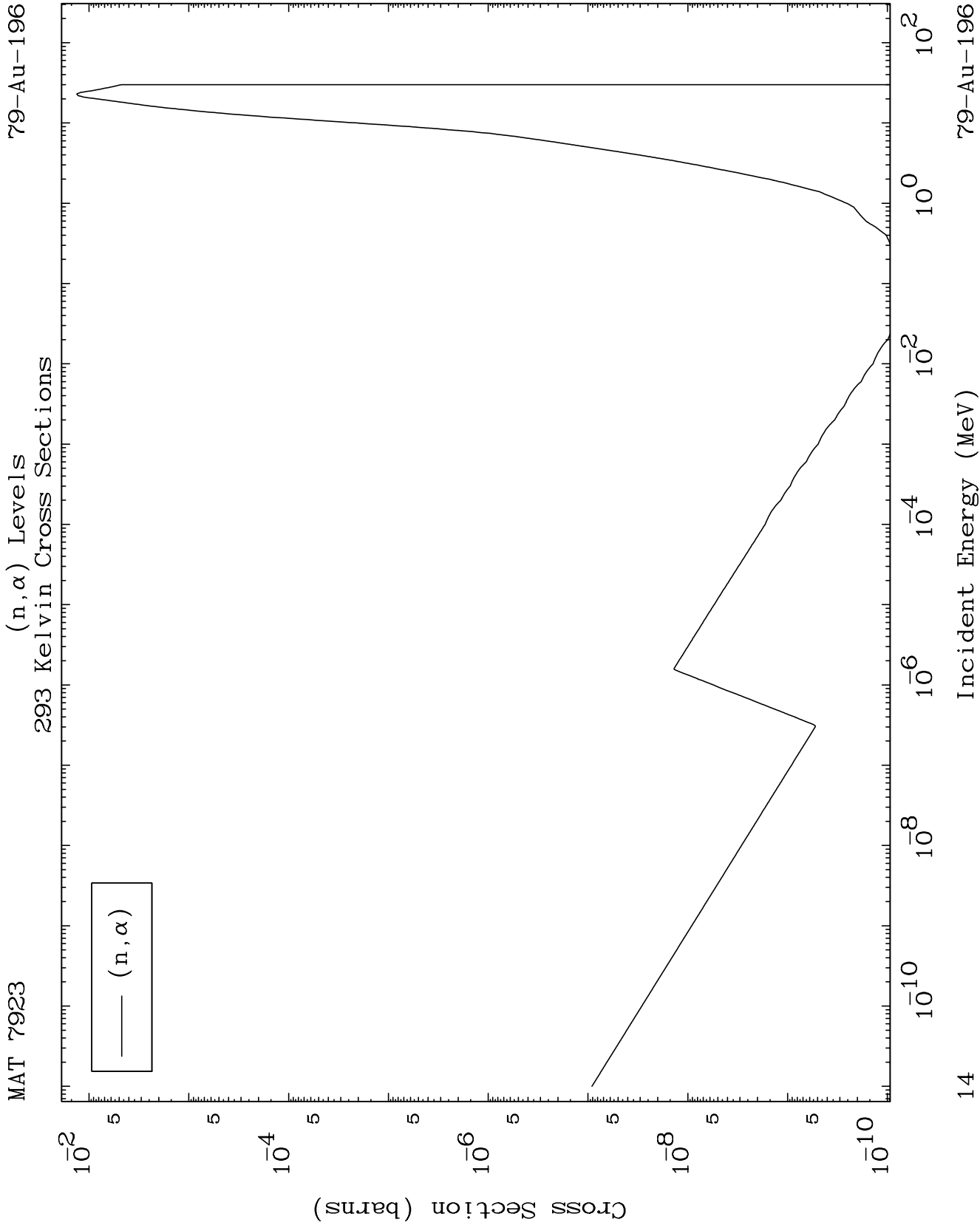


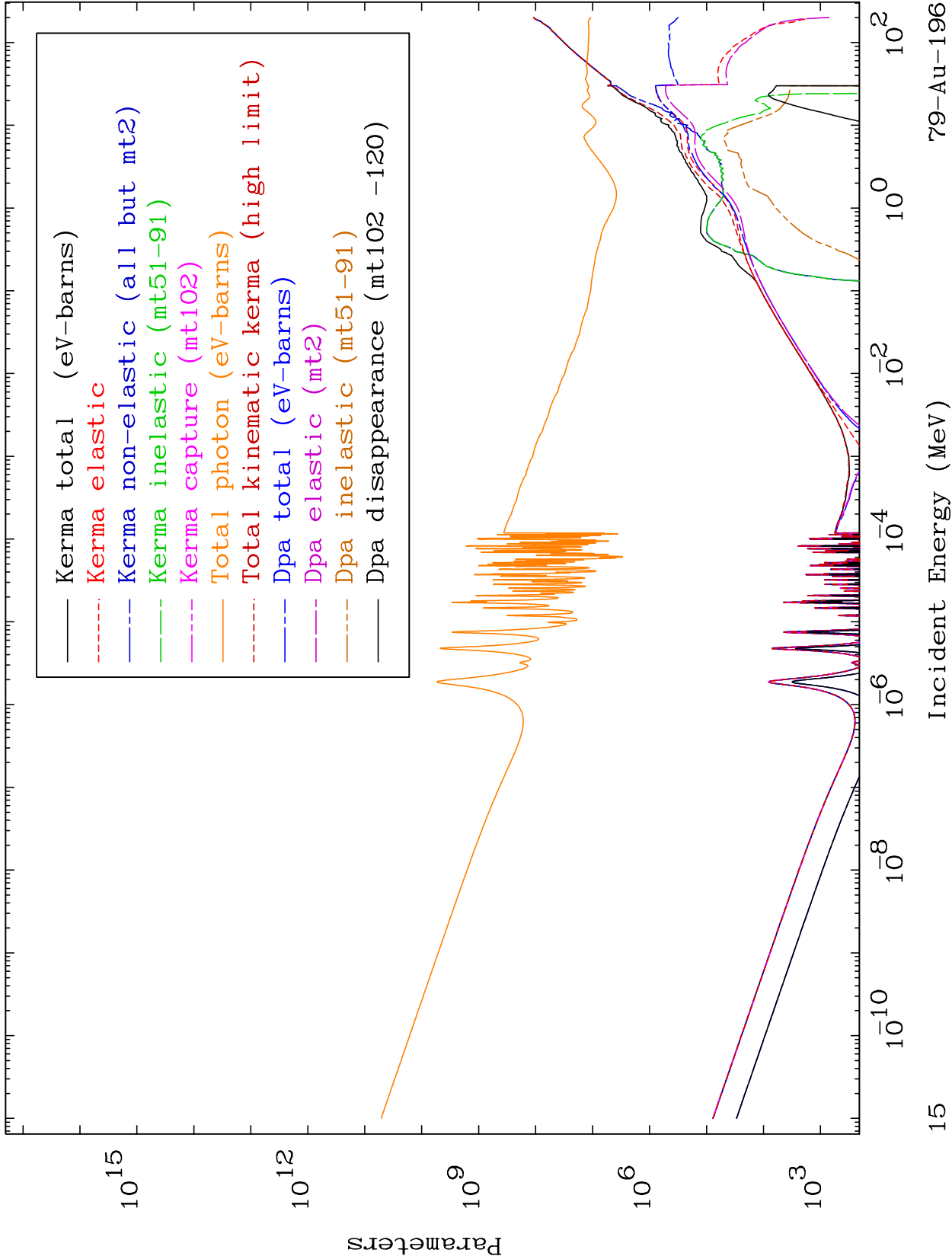
12

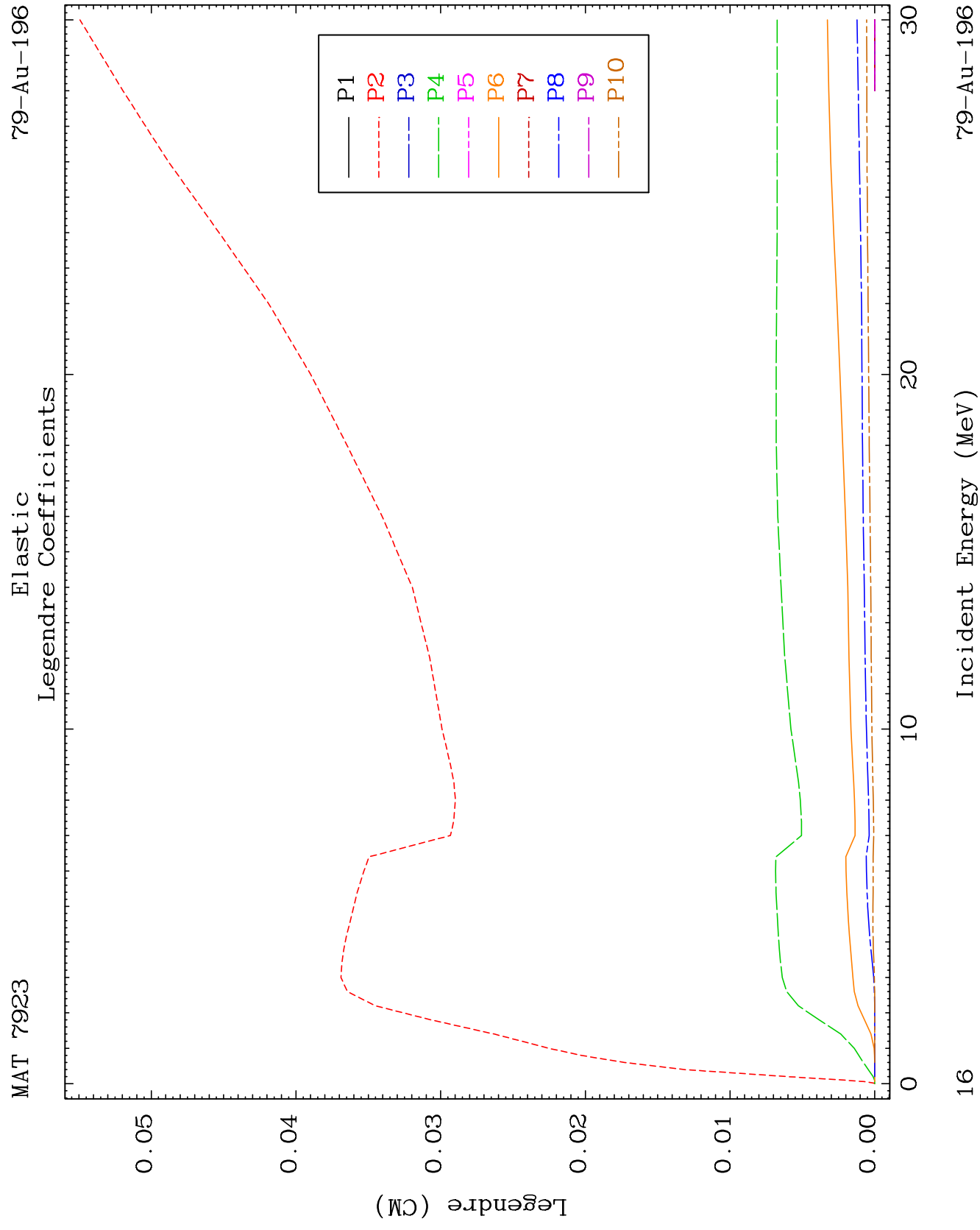
79-Au-196

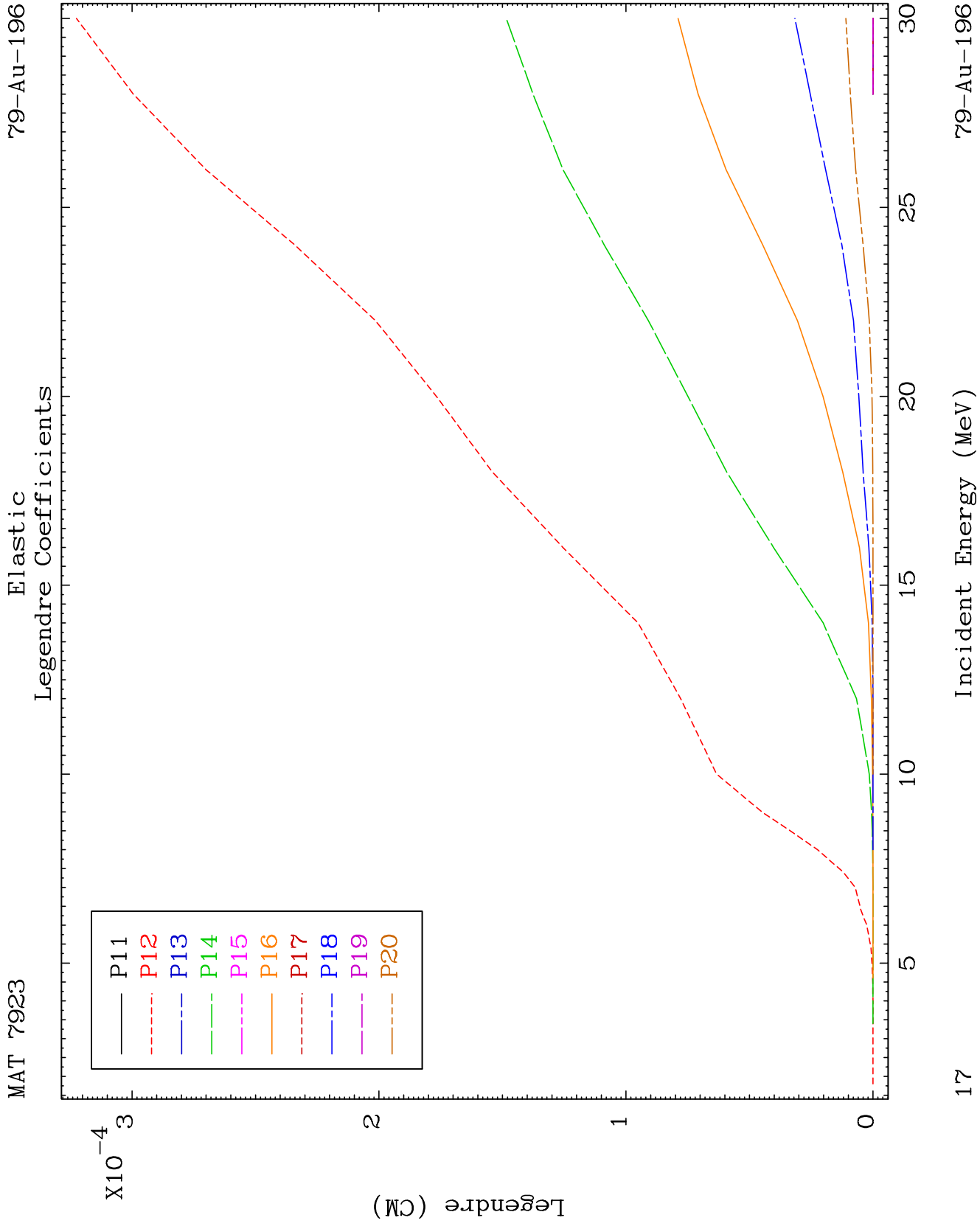
Incident Energy (MeV)

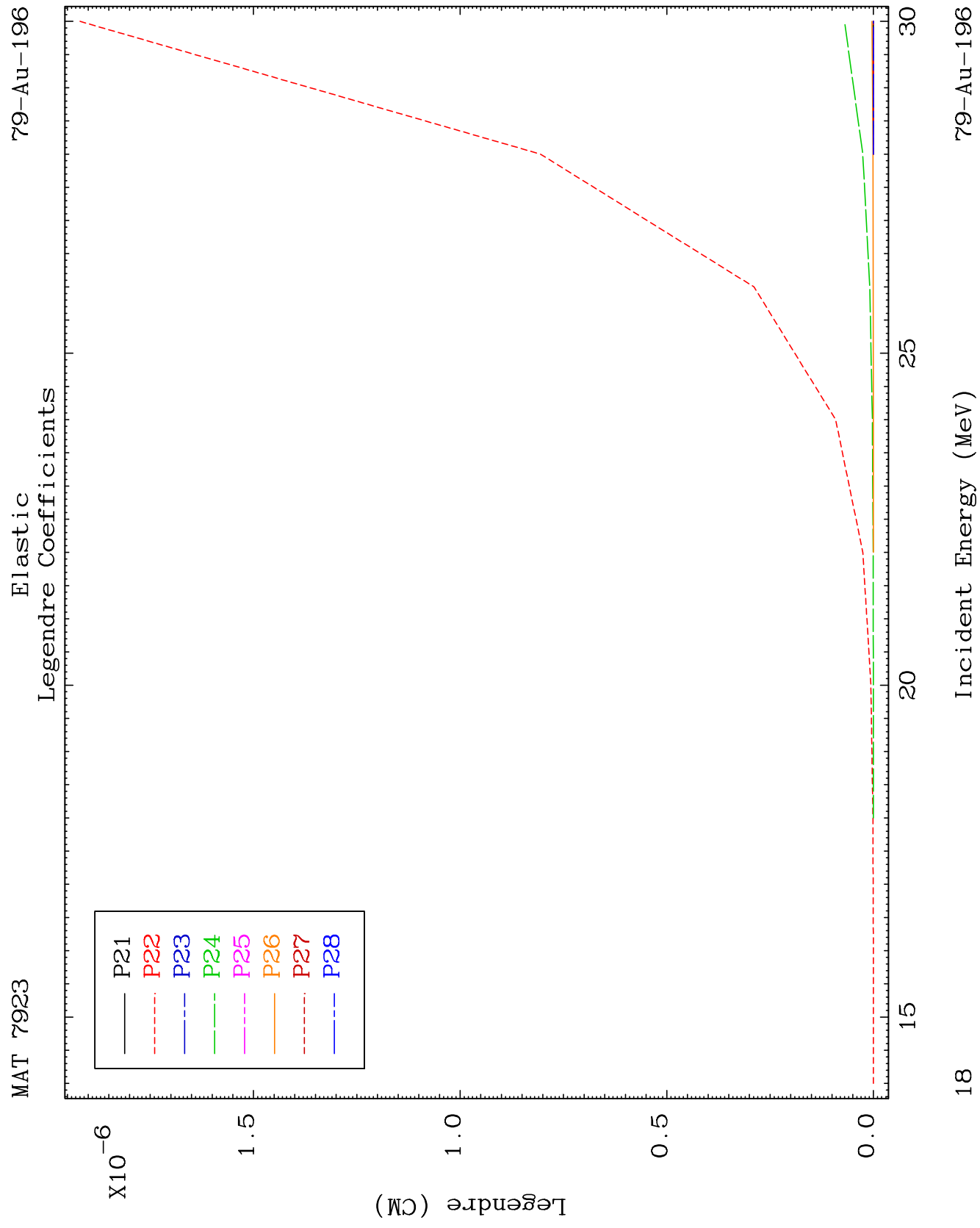








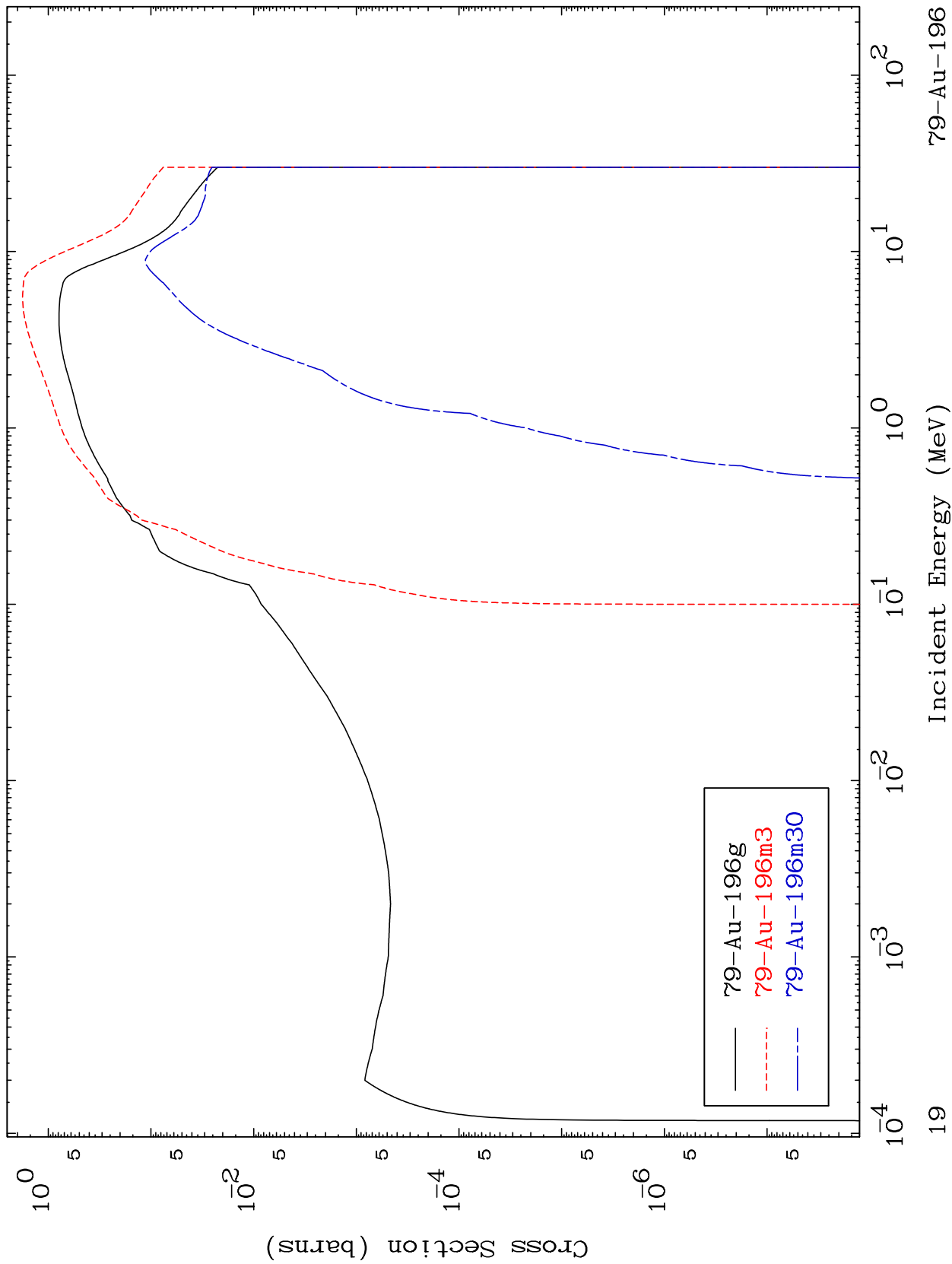




MAT 7923

79-Au-196

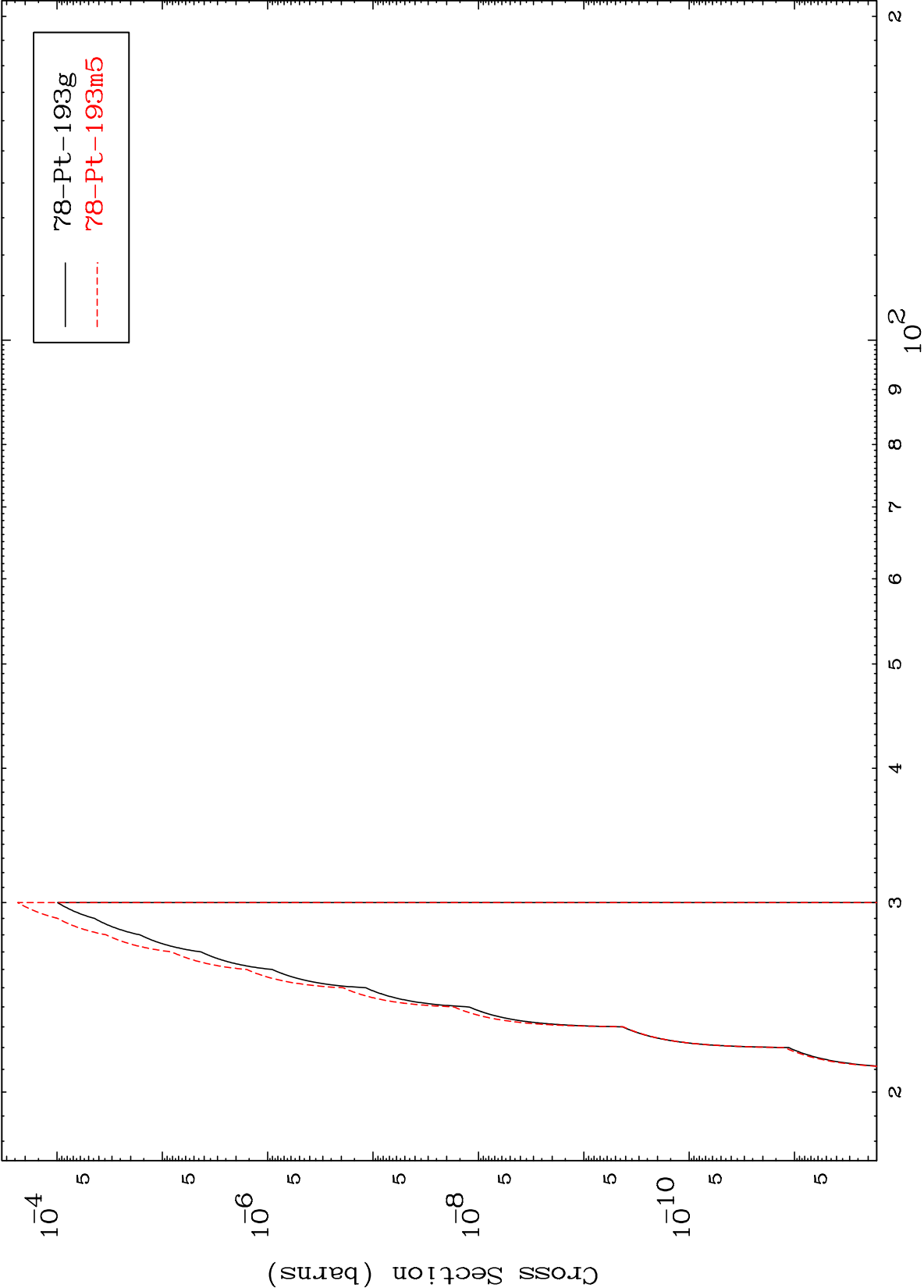
Inelastic  
Radionuclide Production Cross Section



19

79-Au-196

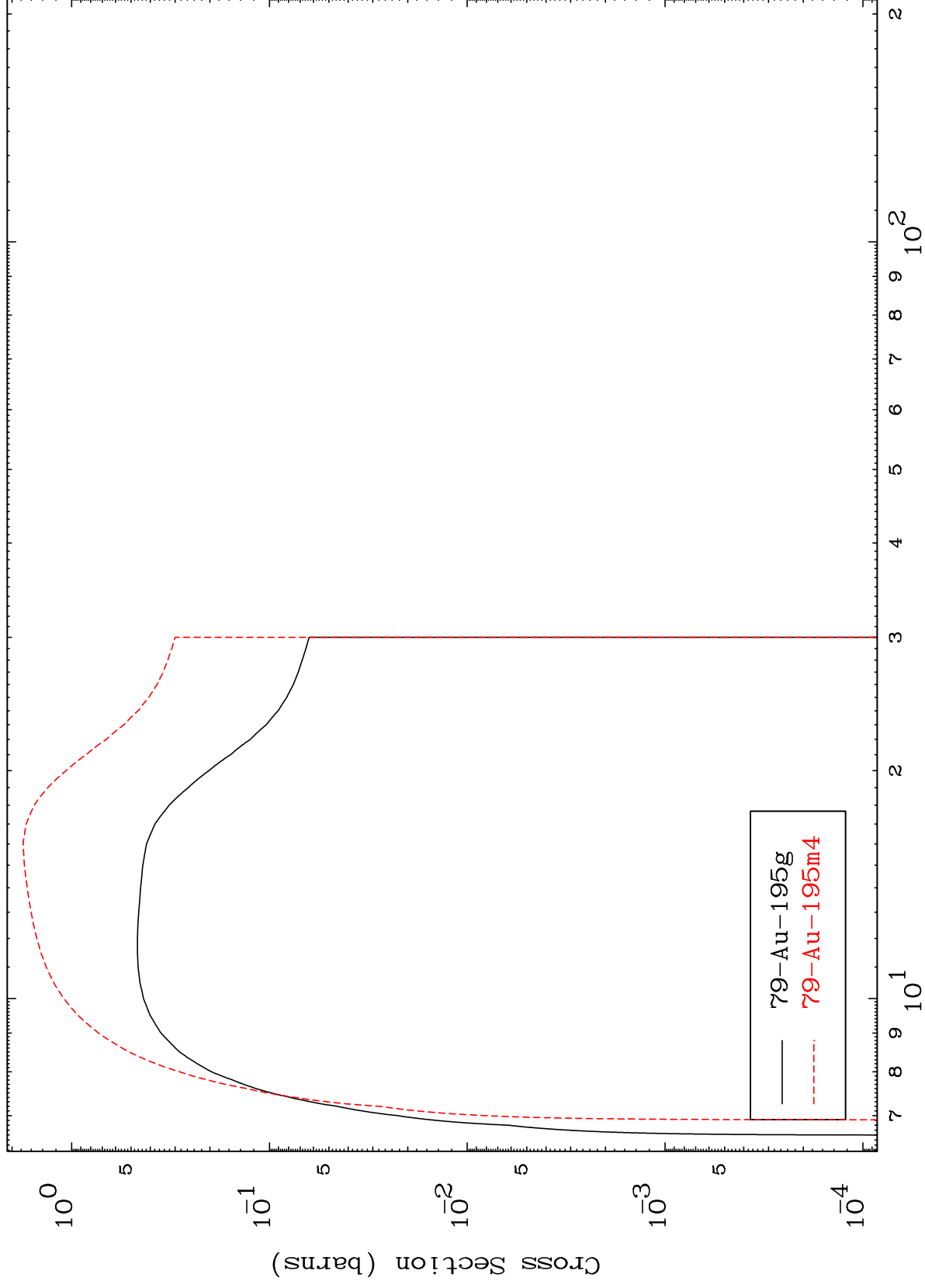
Radionuclide Production Cross Section



MAT 7923

79-Au-196

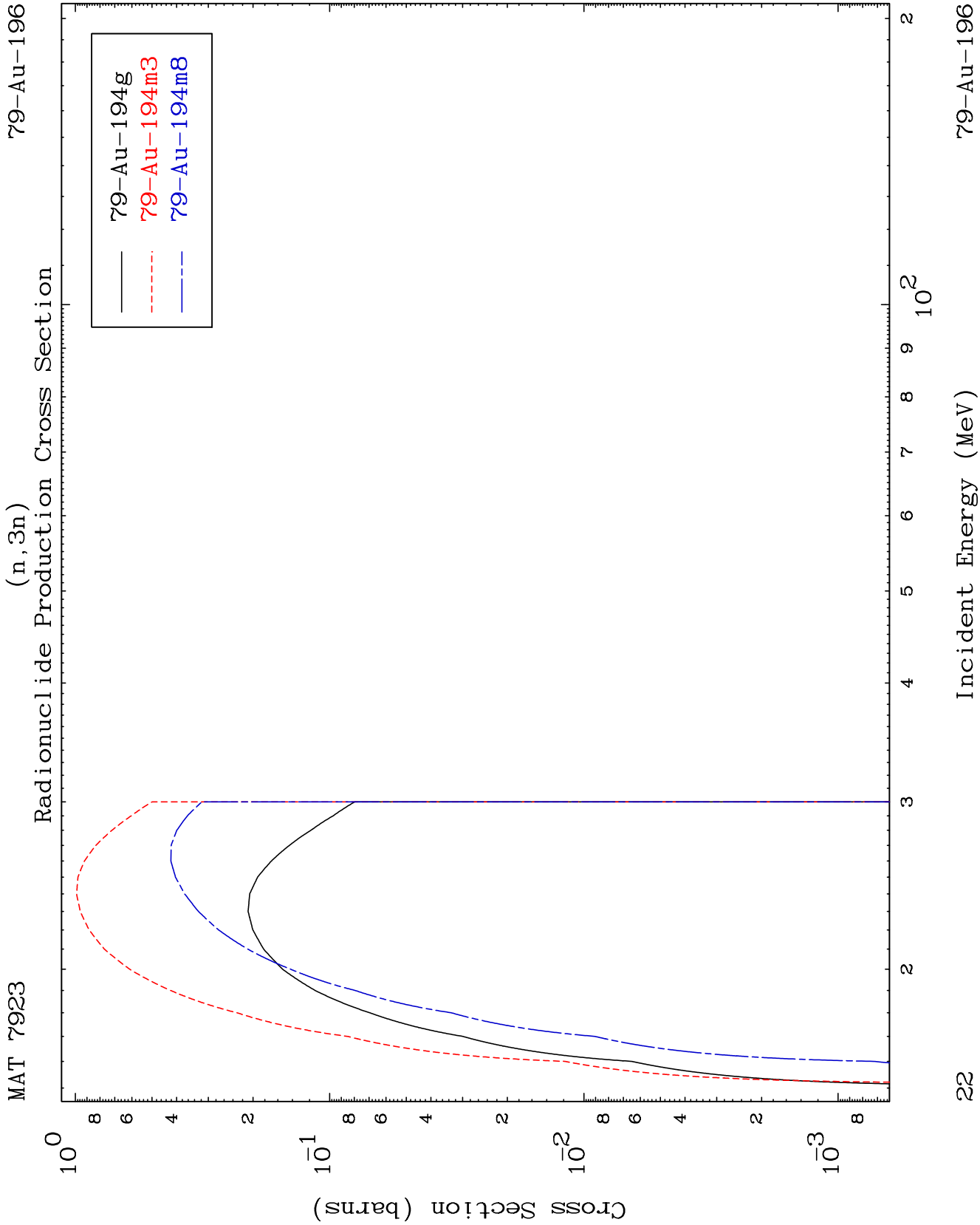
(n,2n)  
Radionuclide Production Cross Section



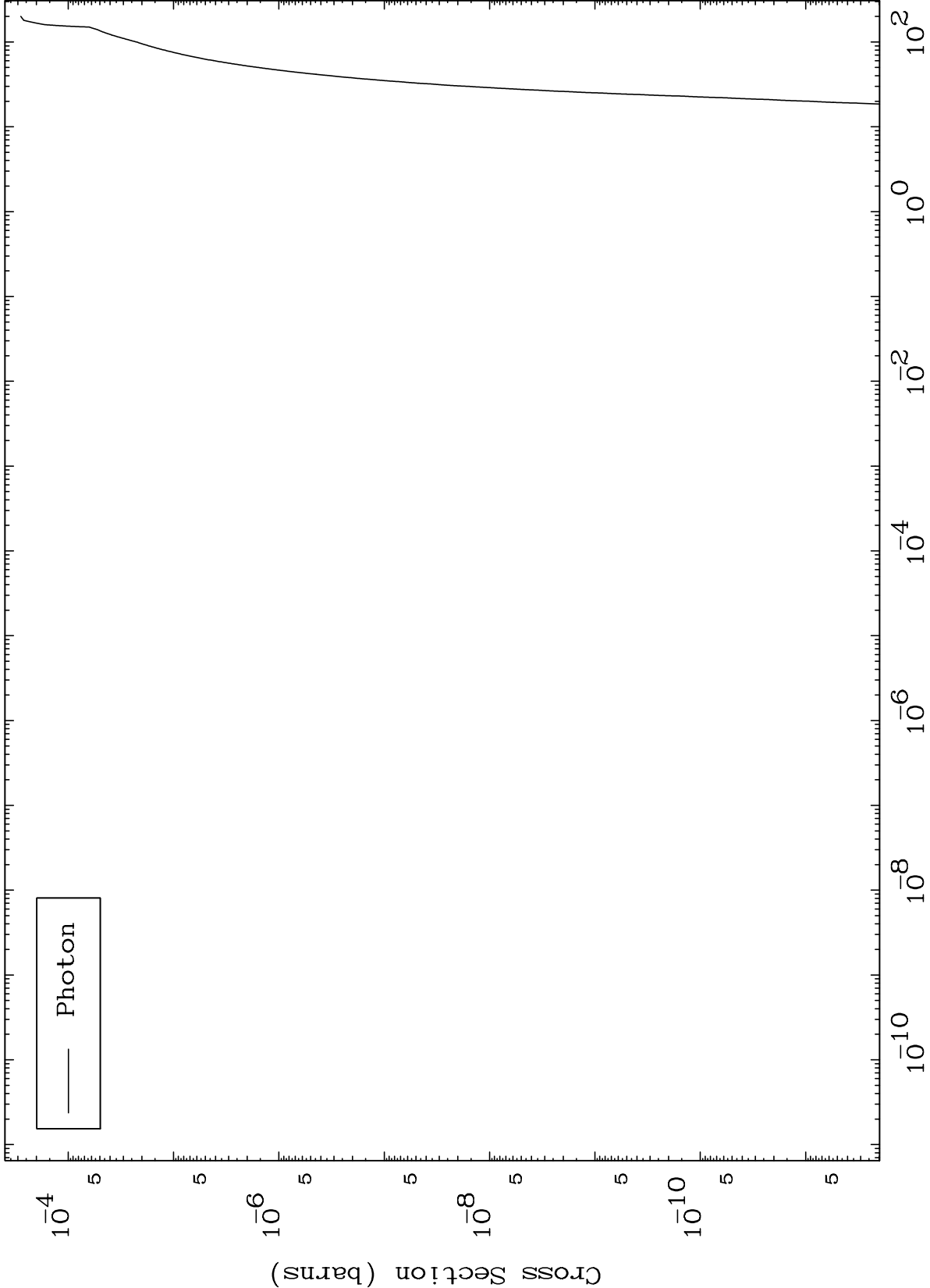
21

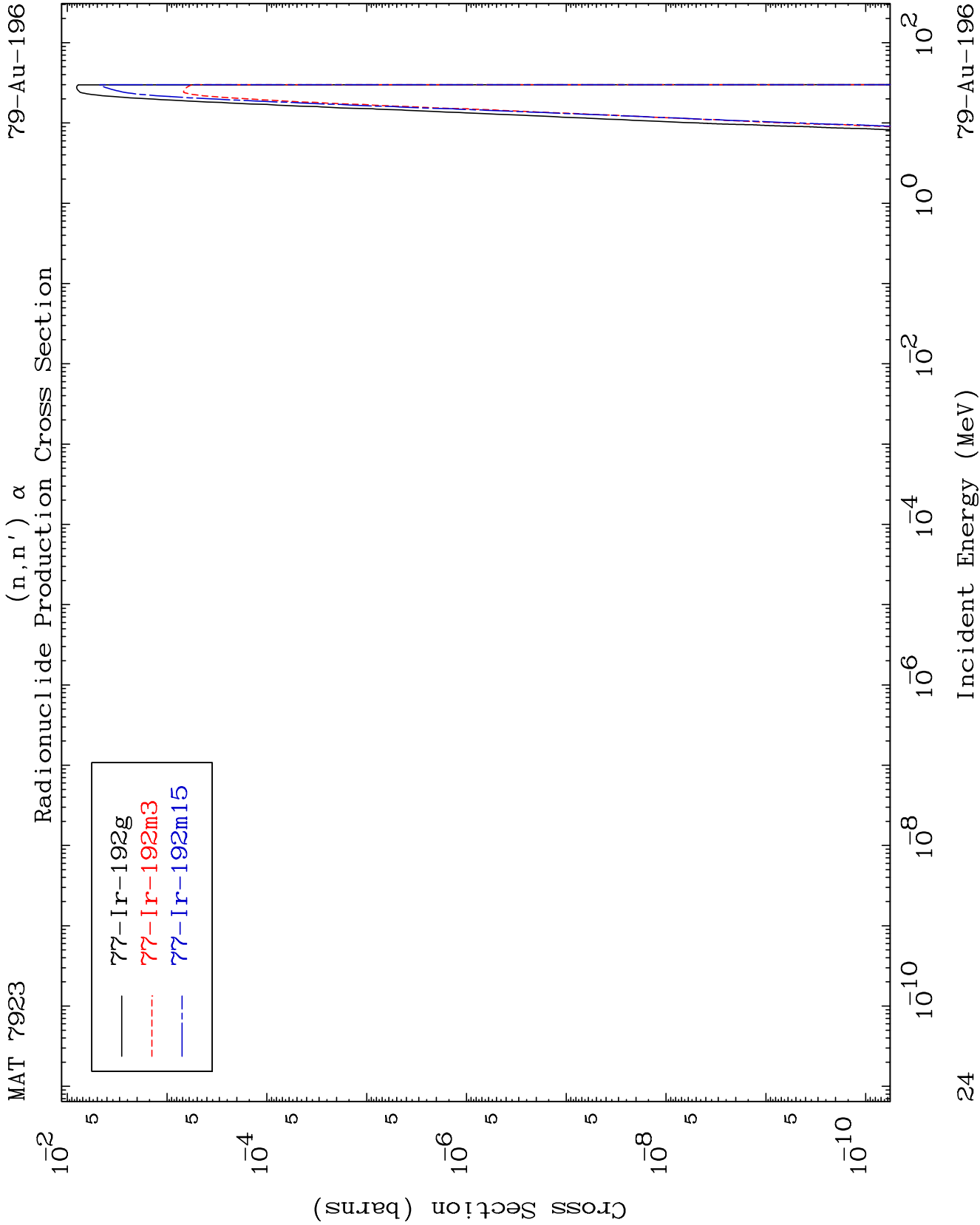
Incident Energy (MeV)

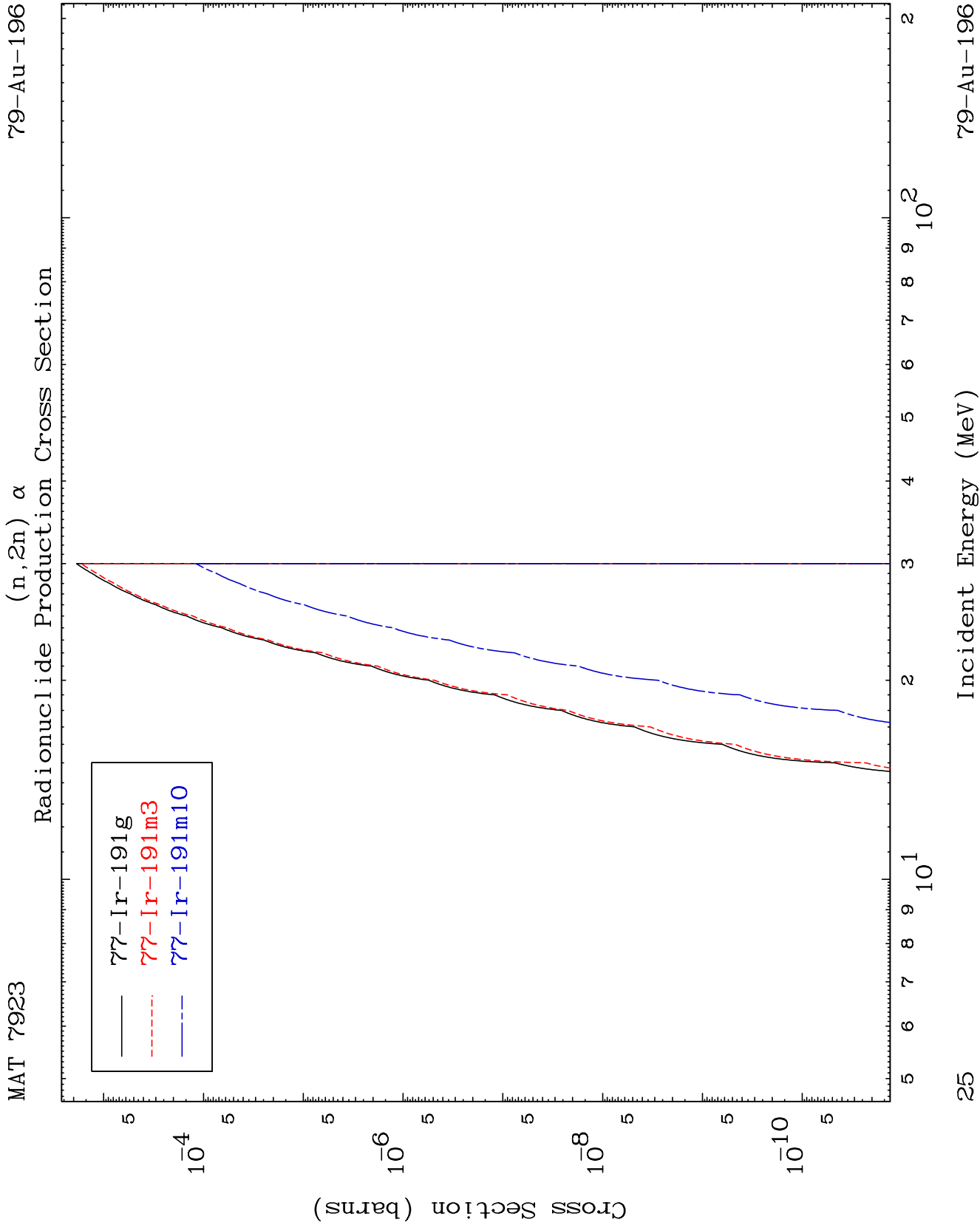
79-Au-196



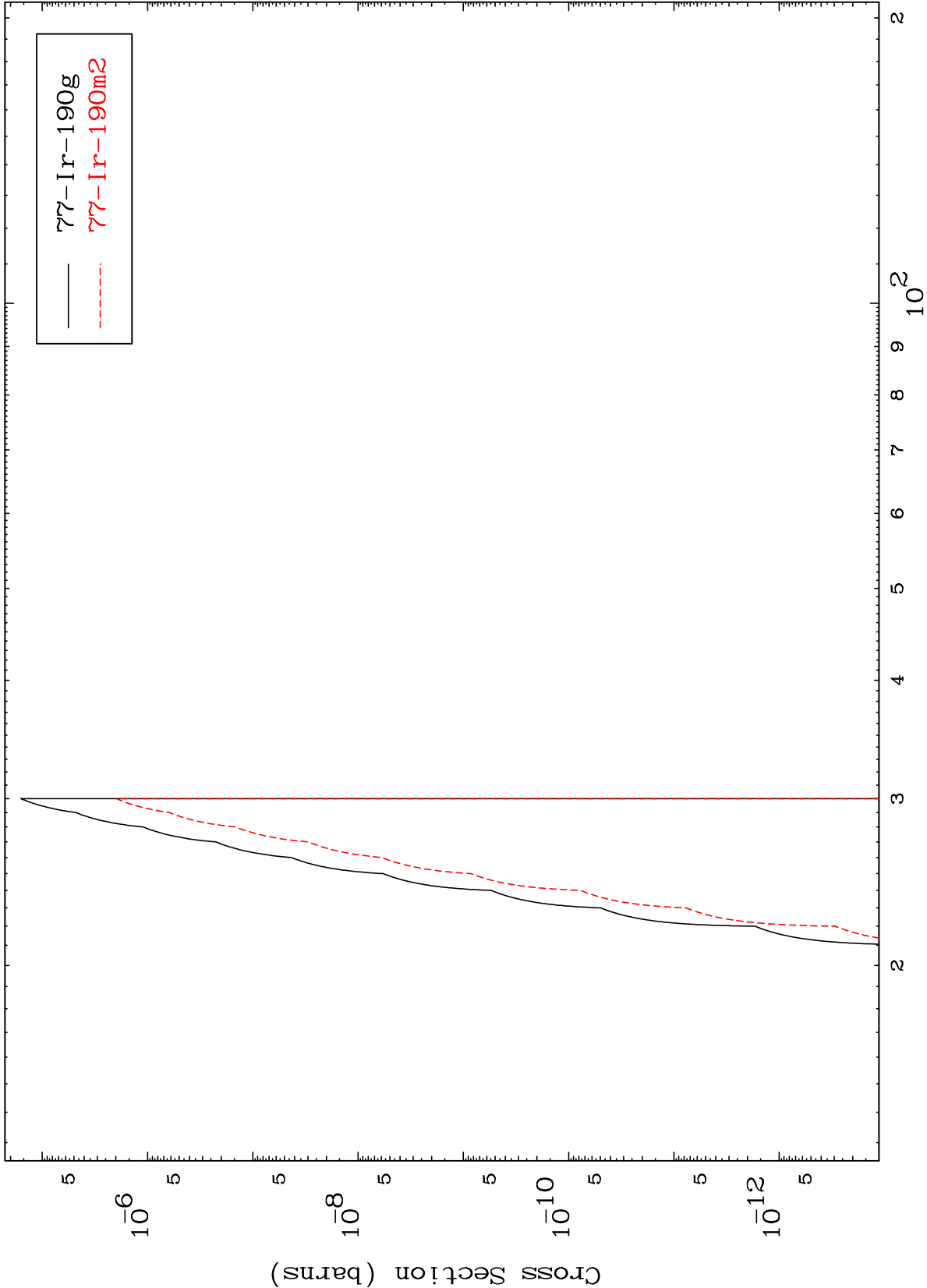
Fission  
Radionuclide Production Cross Section







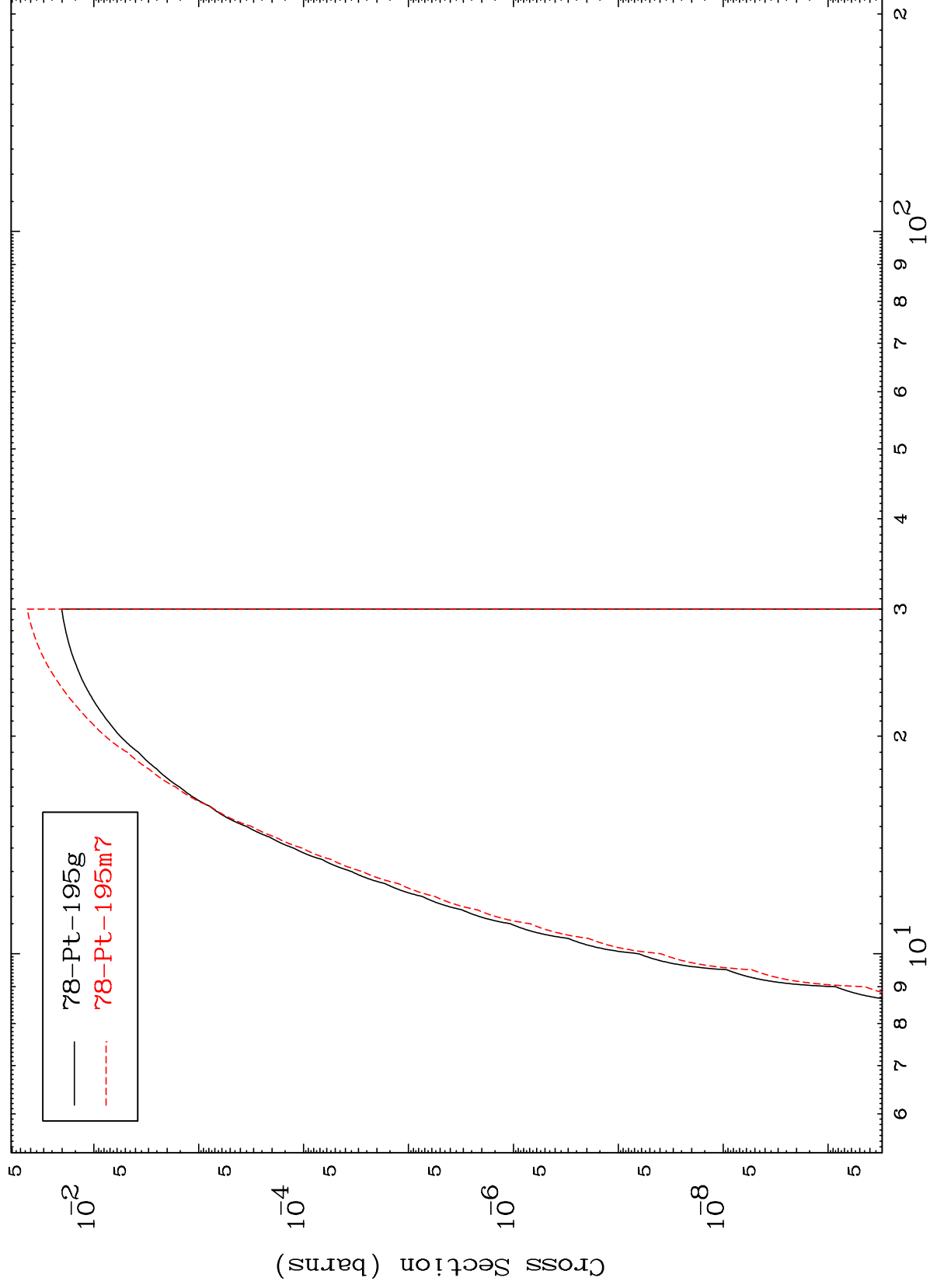
Radionuclide Production Cross Section



MAT 7923

79-Au-196

(n,n') p  
Radionuclide Production Cross Section

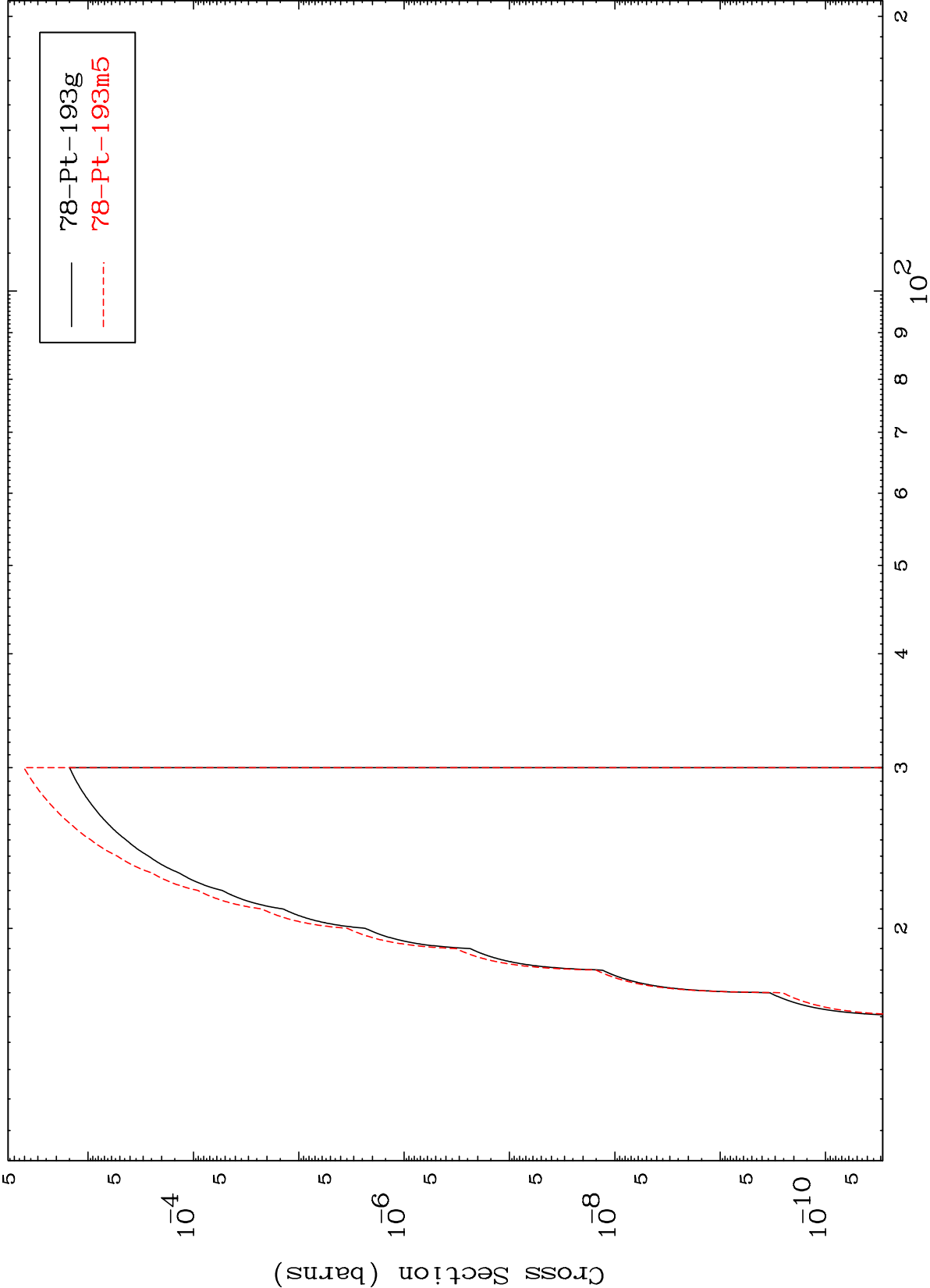


27

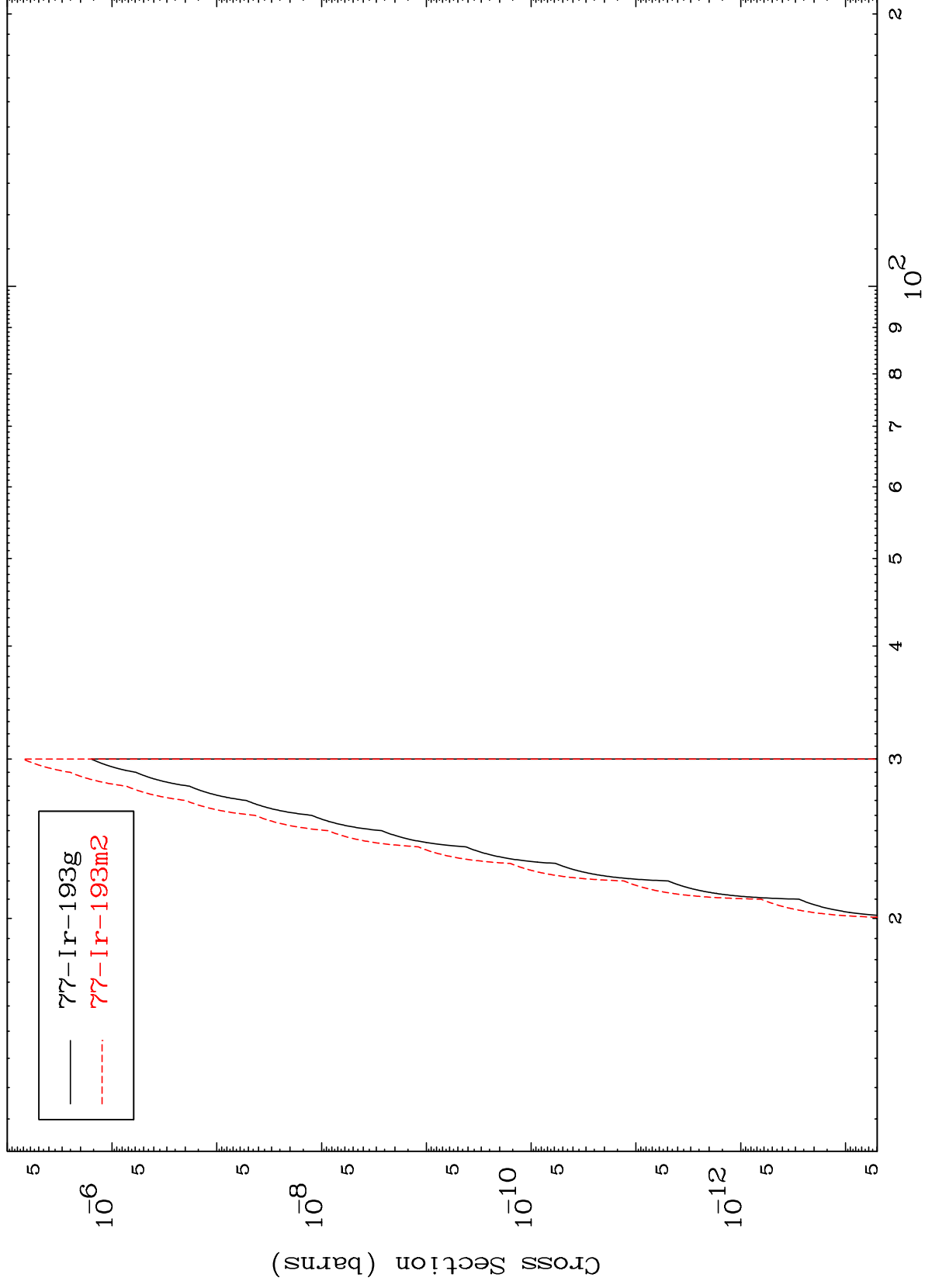
Incident Energy (MeV)

79-Au-196

Radionuclide Production Cross Section



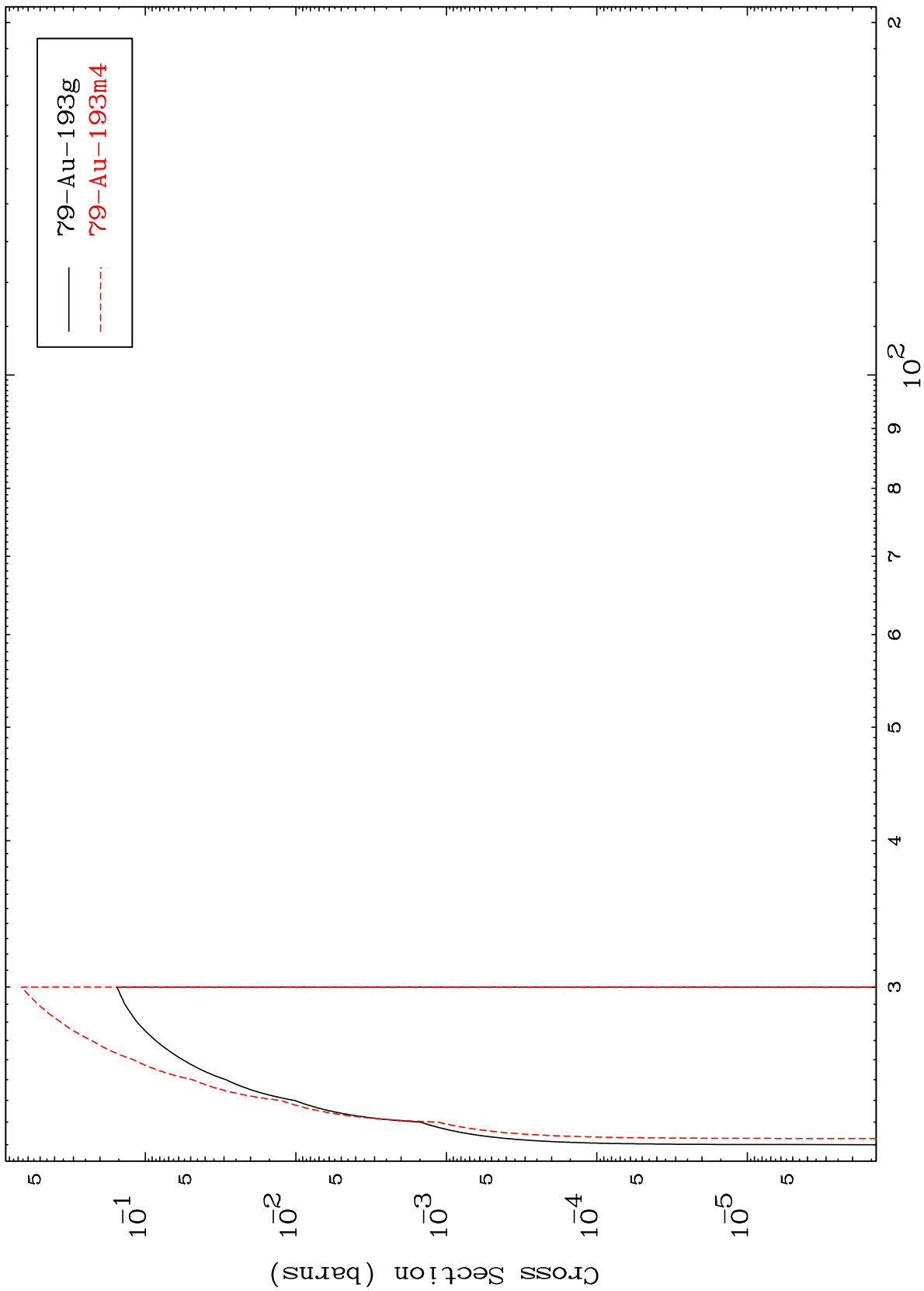
Radionuclide Production Cross Section



MAT 7923

79-Au-196

(n,4n)  
Radionuclide Production Cross Section

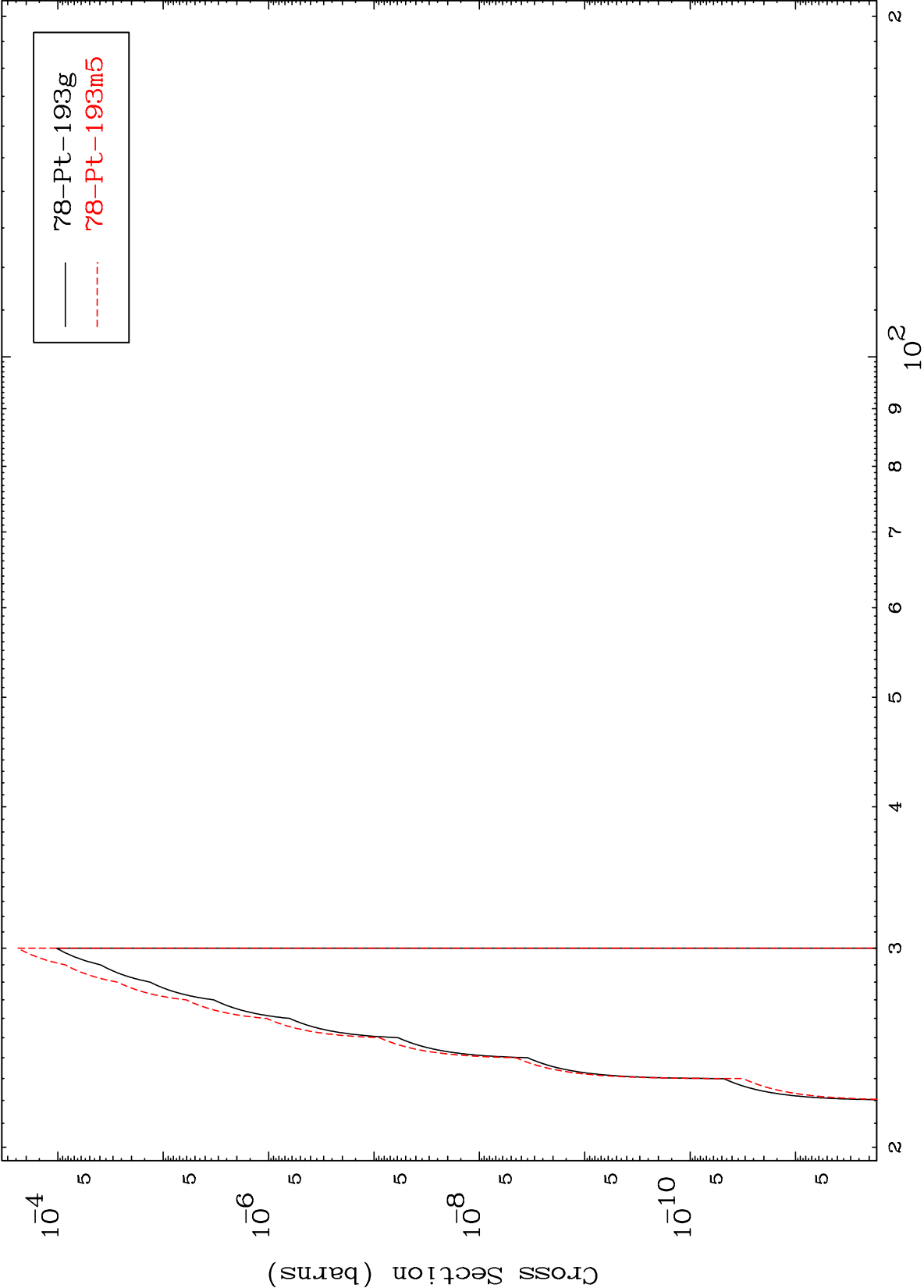


79-Au-196

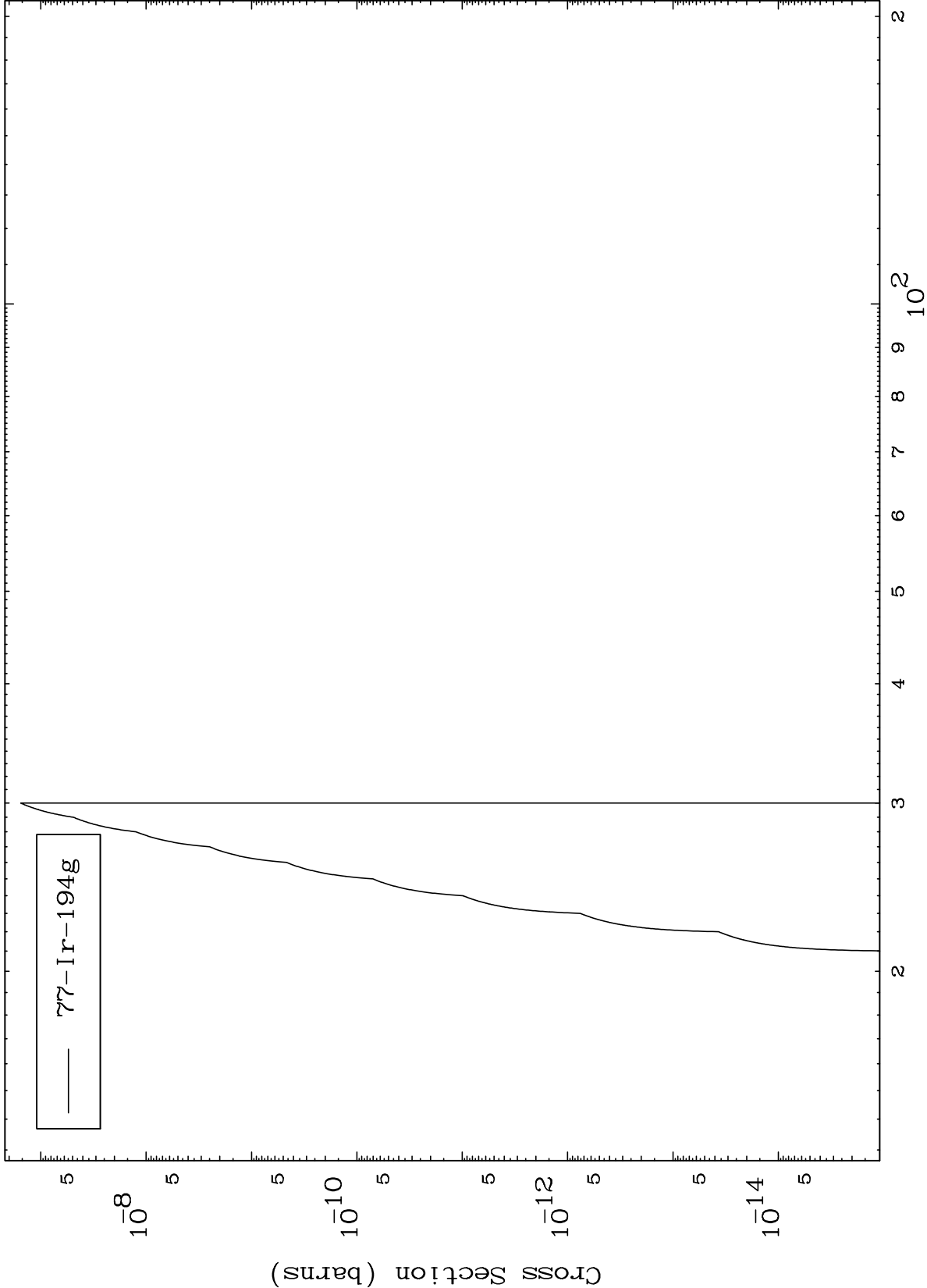
Incident Energy (MeV)

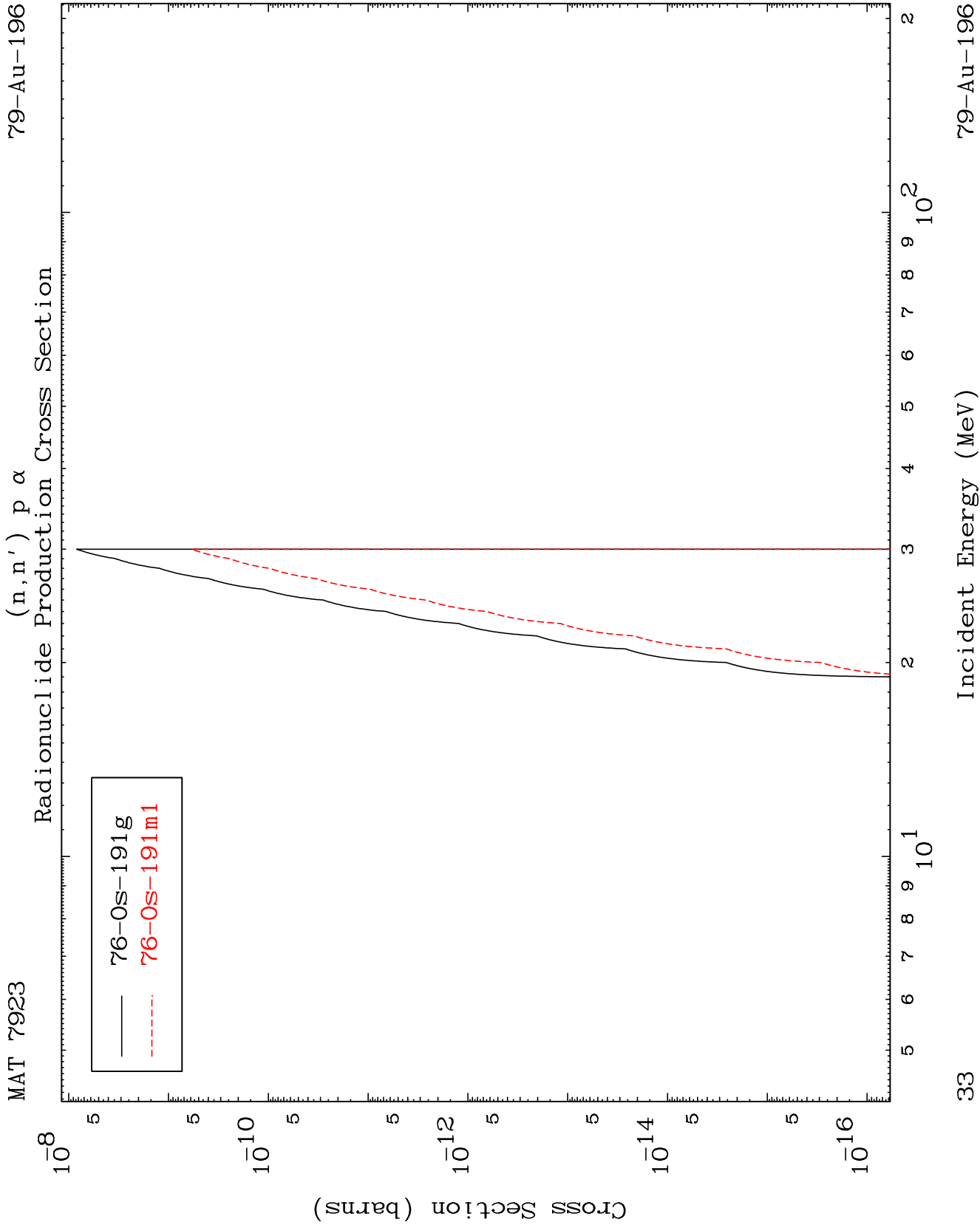
30

(n,3n) p  
Radionuclide Production Cross Section



(n,2n) p  
Radionuclide Production Cross Section

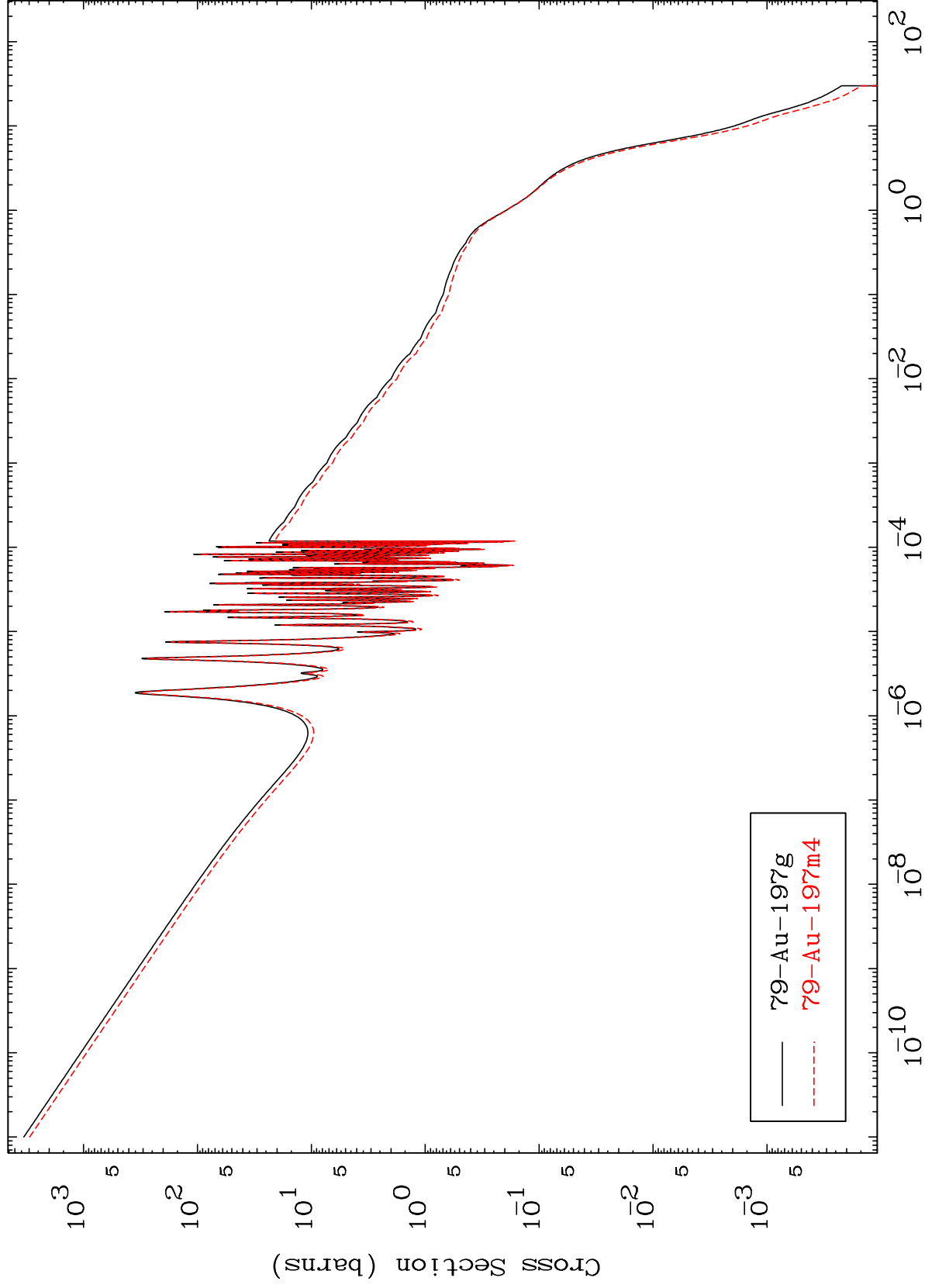




MAT 7923

$^{79}\text{Au-196}$

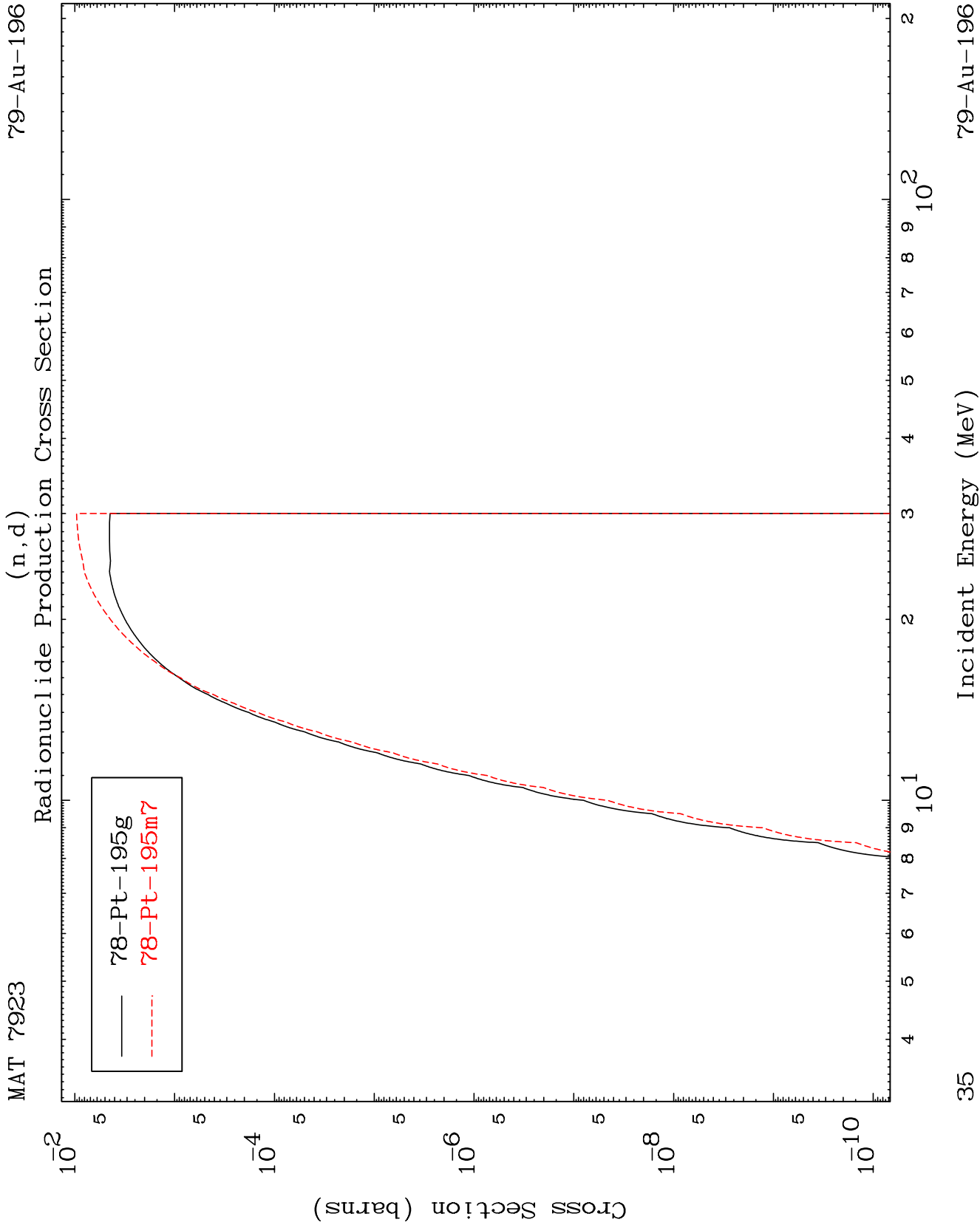
$(n,\gamma)$   
Radionuclide Production Cross Section



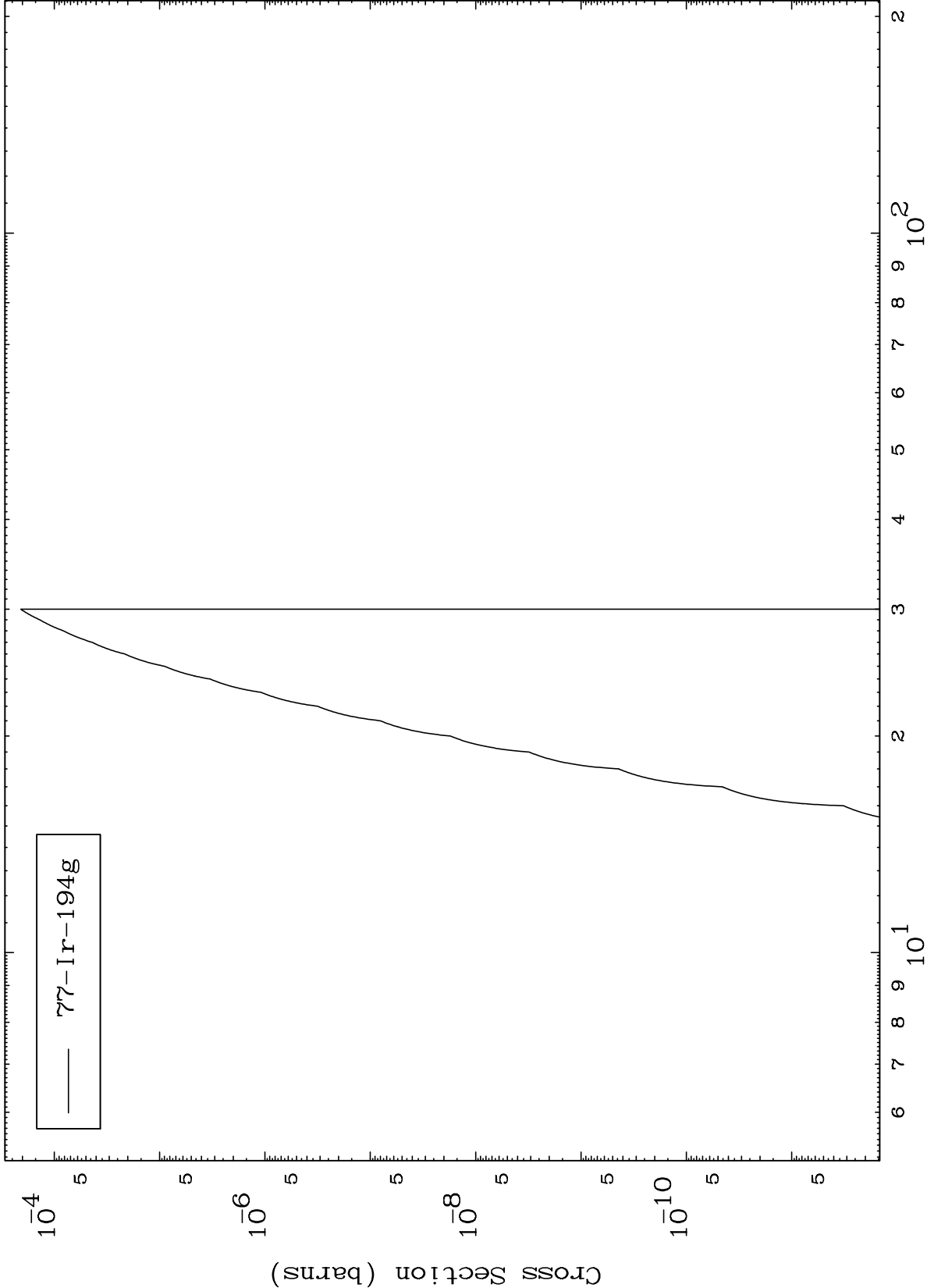
$^{79}\text{Au-196}$

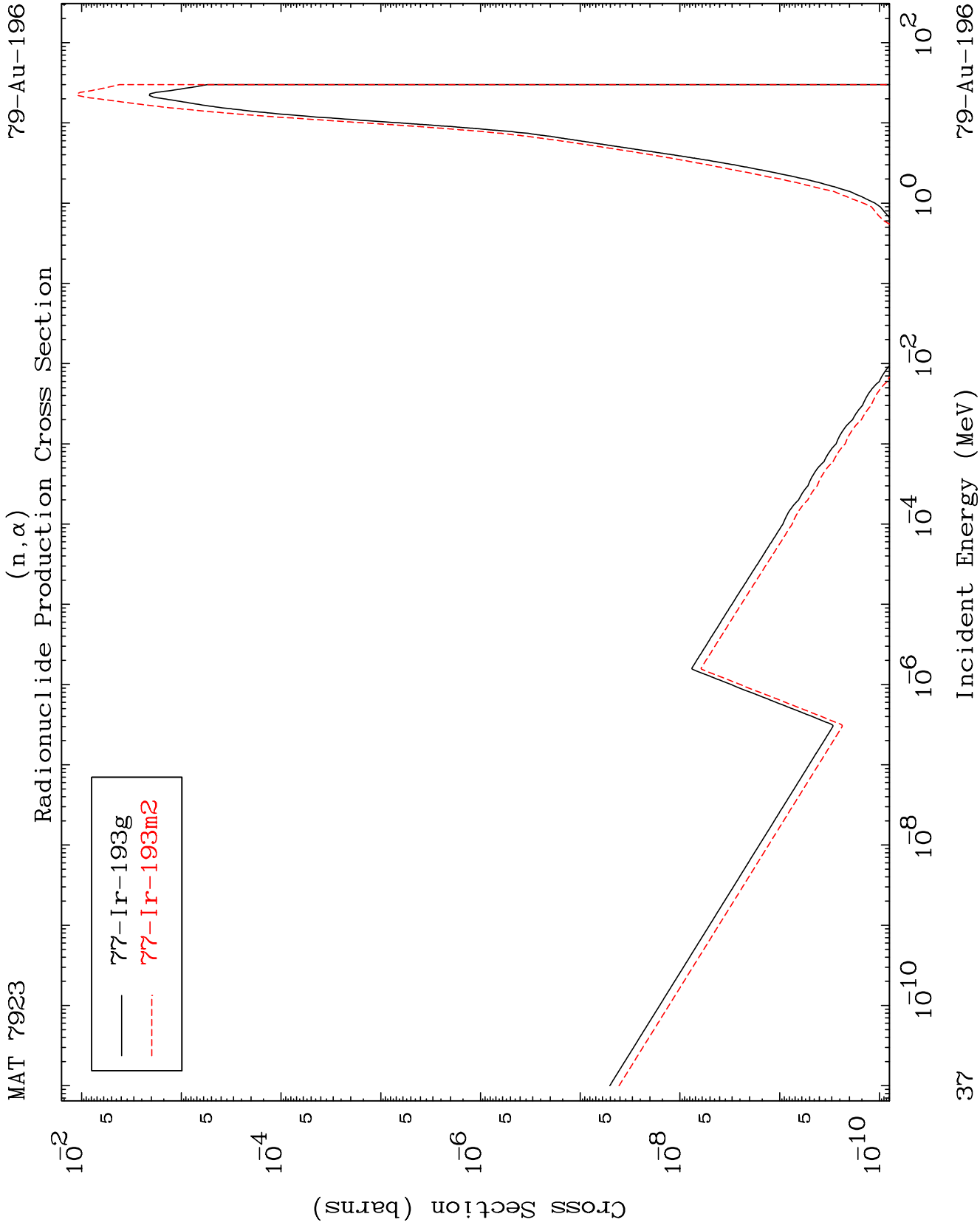
Incident Energy (MeV)

34



Radionuclide Production Cross Section

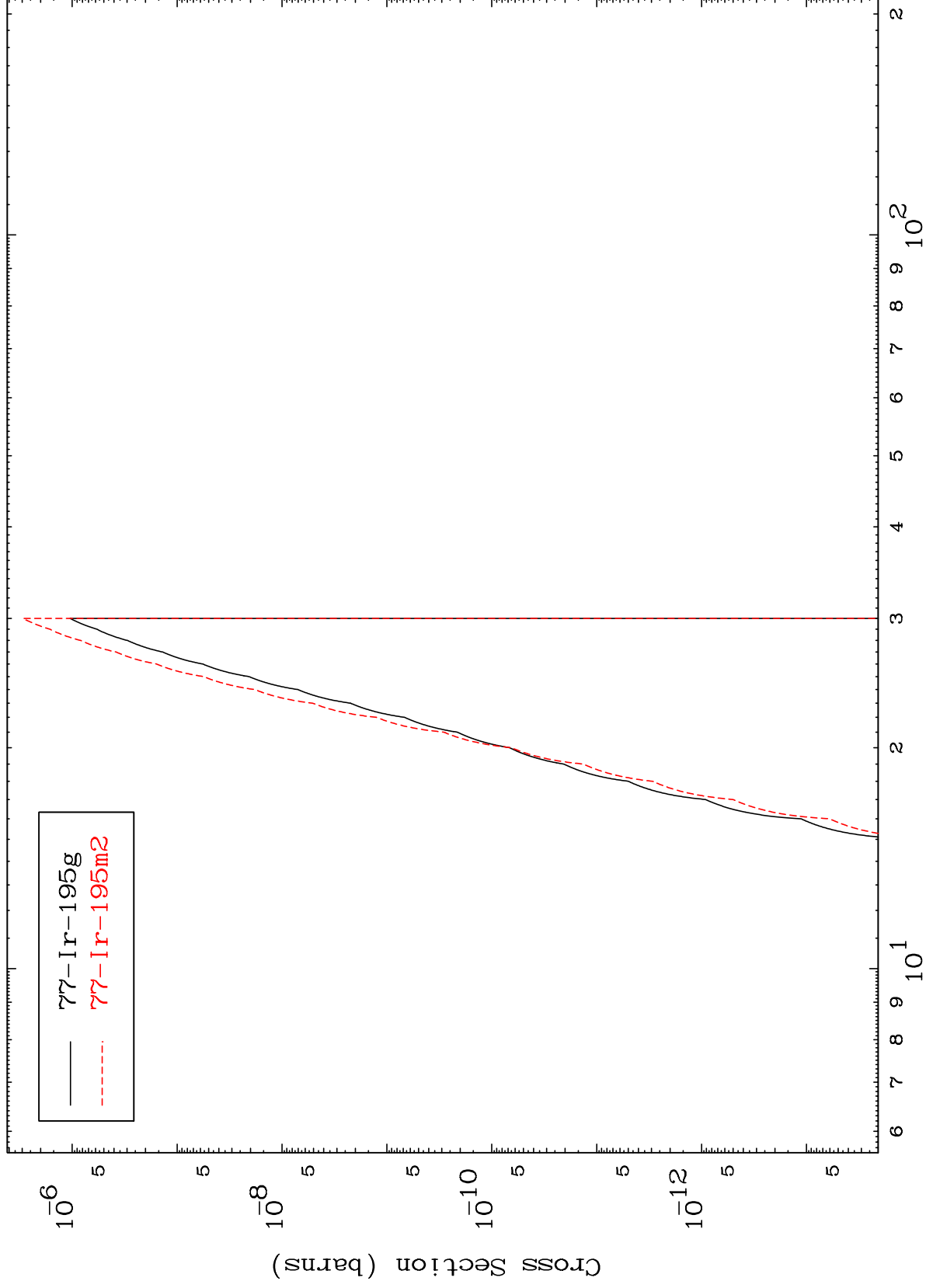




MAT 7923

79-Au-196

(n,2p)  
Radionuclide Production Cross Section

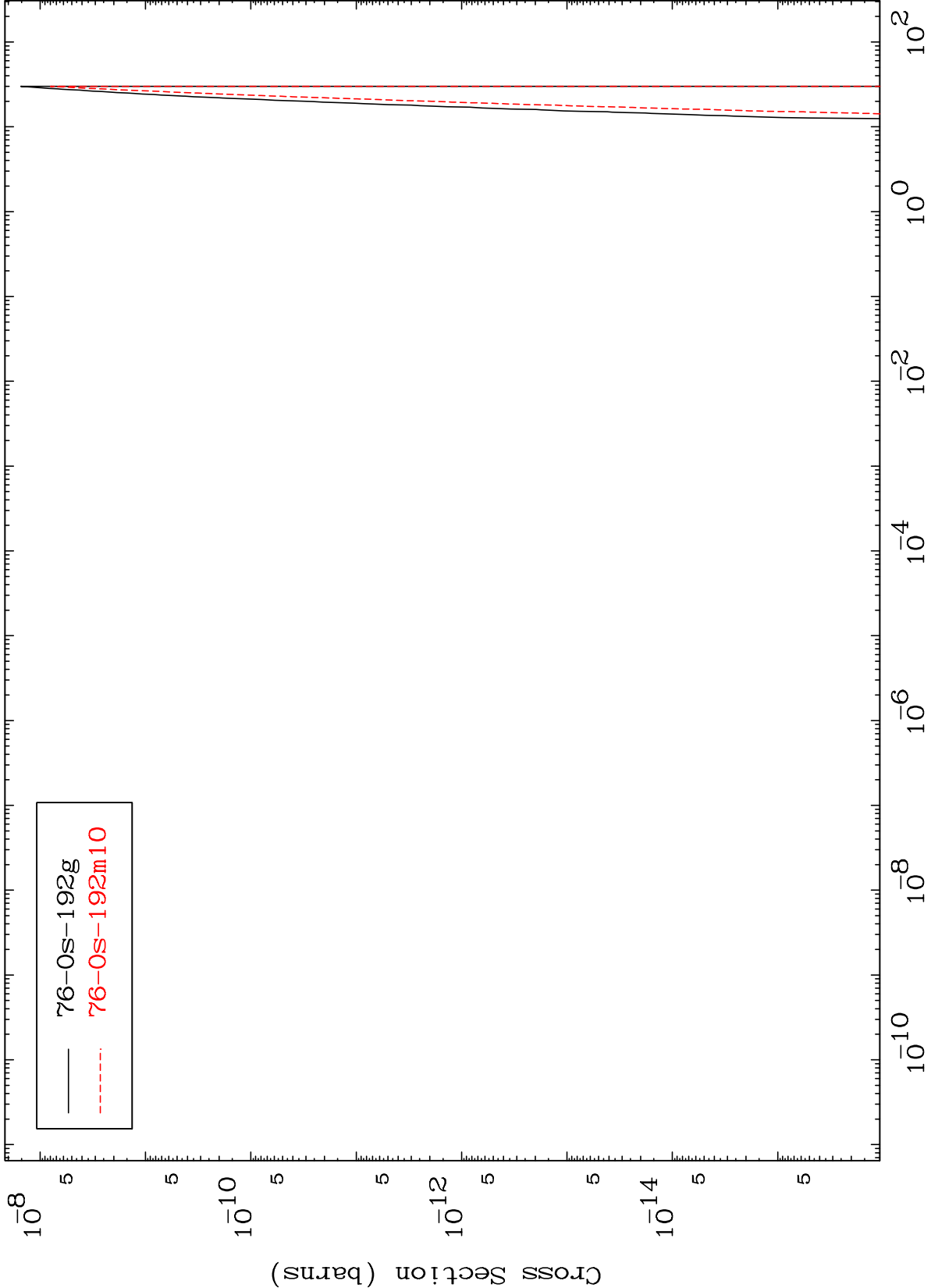


38

Incident Energy (MeV)

79-Au-196

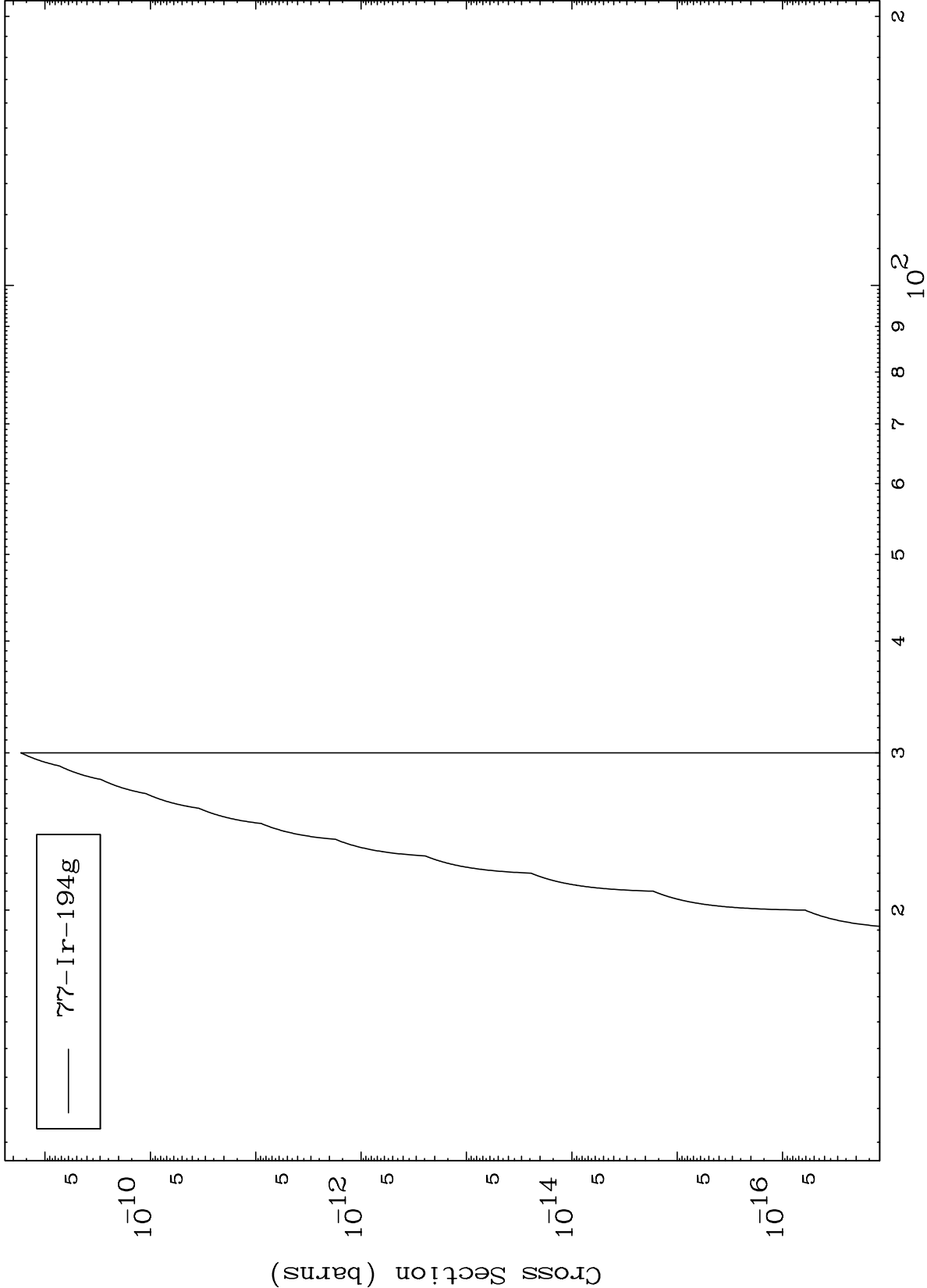
Radionuclide Production Cross Section



MAT 7923

79-Au-196

(n,p) d  
Radionuclide Production Cross Section



40

Incident Energy (MeV)

79-Au-196

(n,p) t  
Radionuclide Production Cross Section

