

Program EVALPLOT
(Version 2017-1)

by

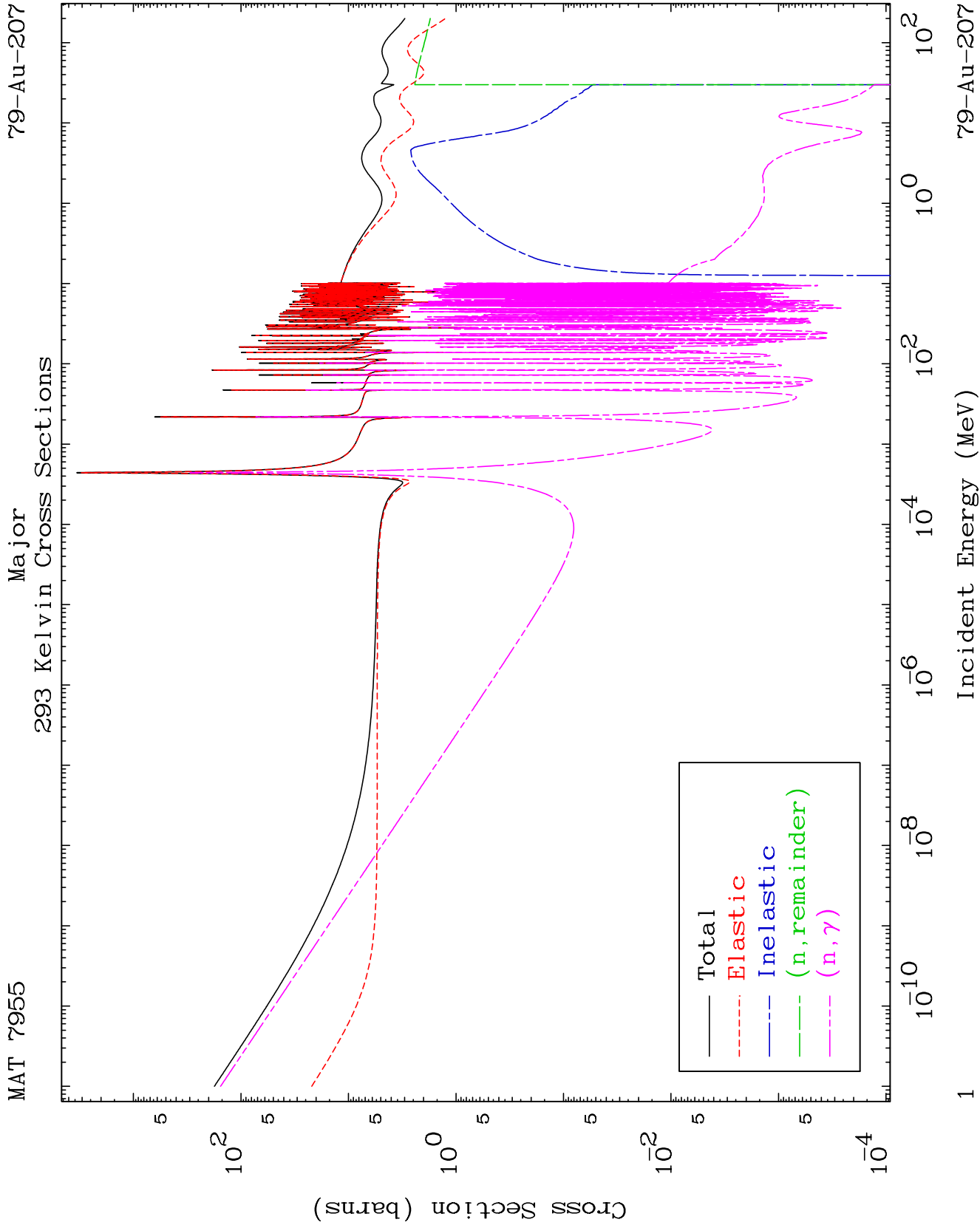
Dermott E. Cullen
(Present Contact Information)

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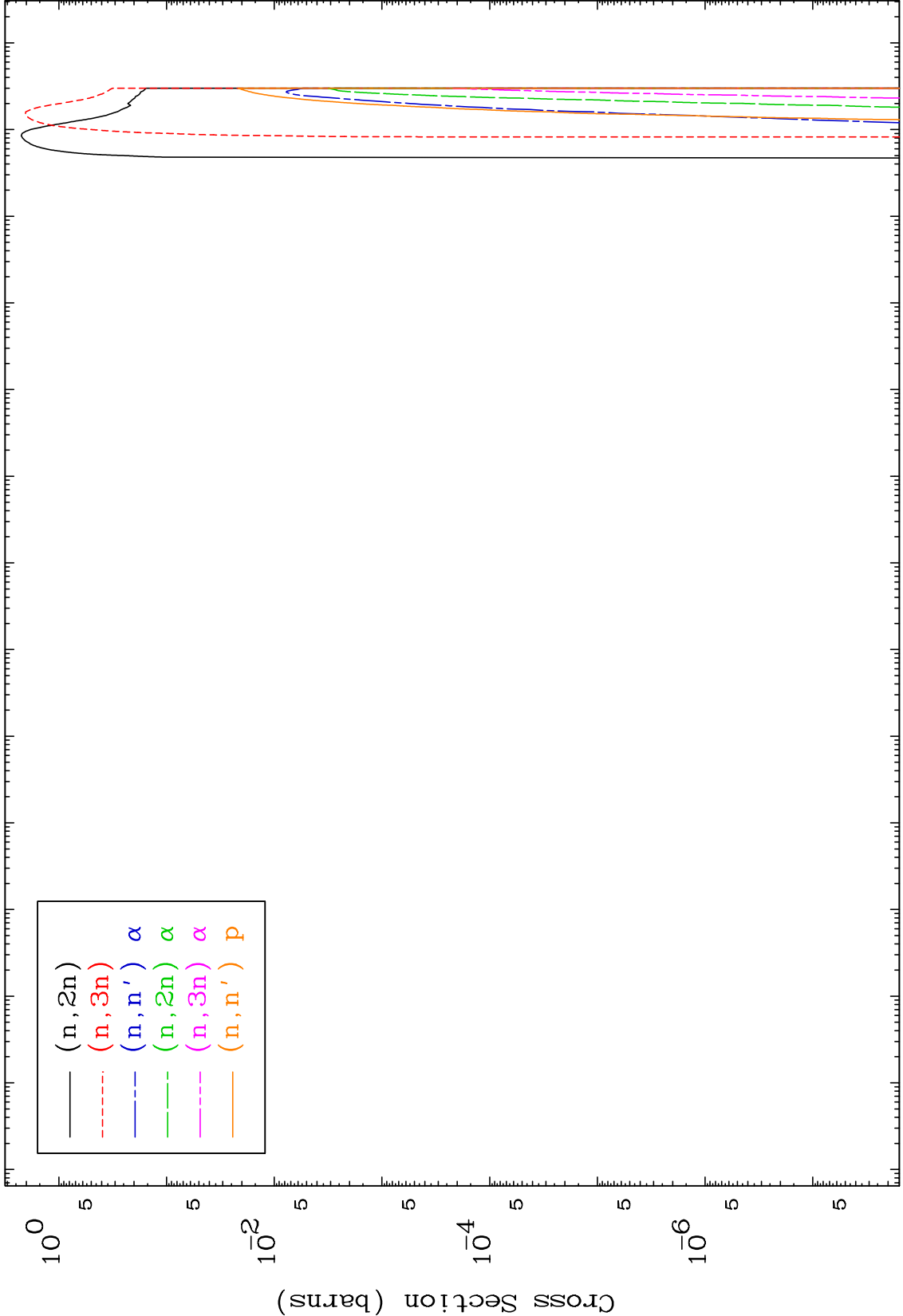
Press Mouse Button to Start

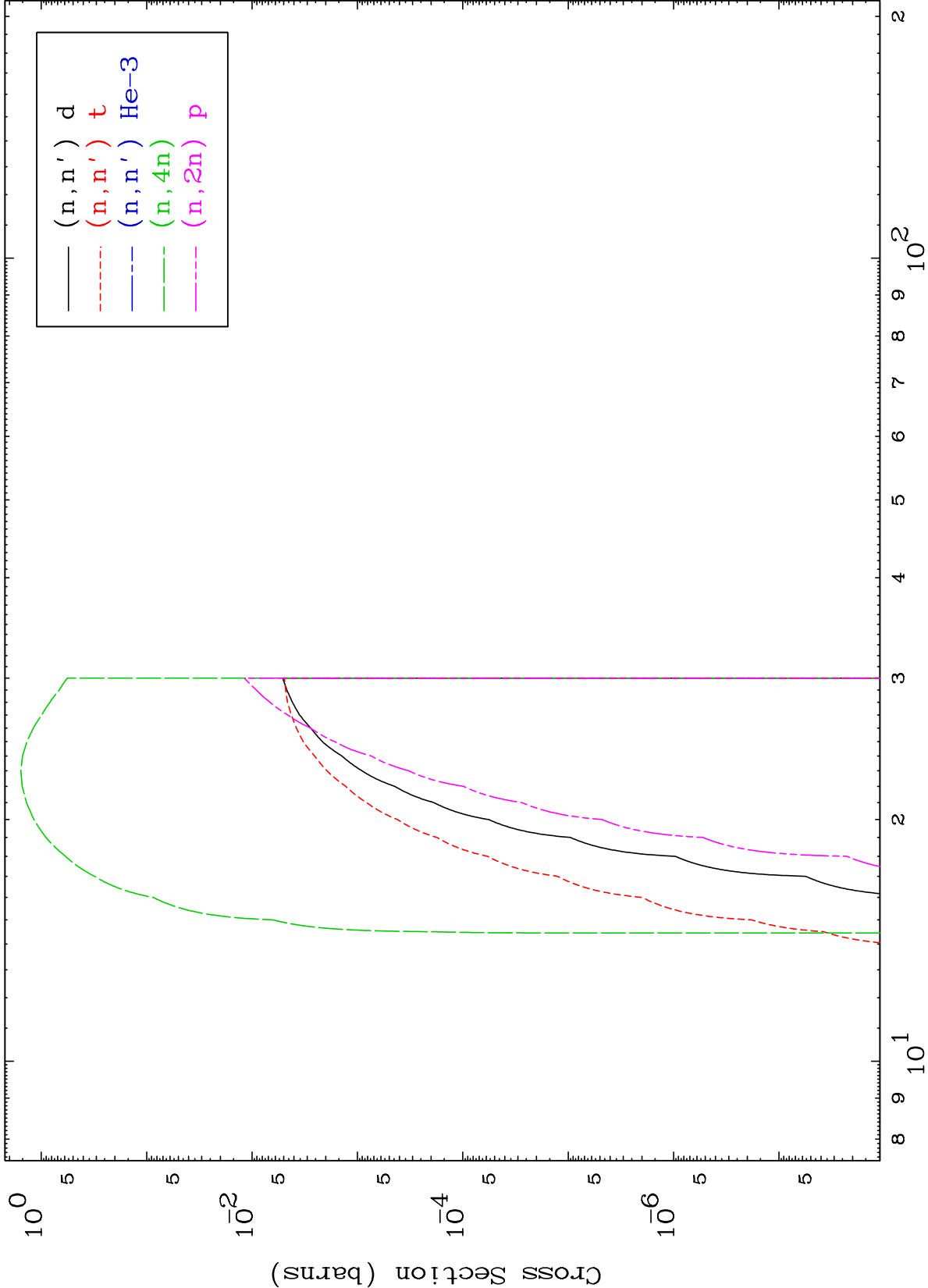


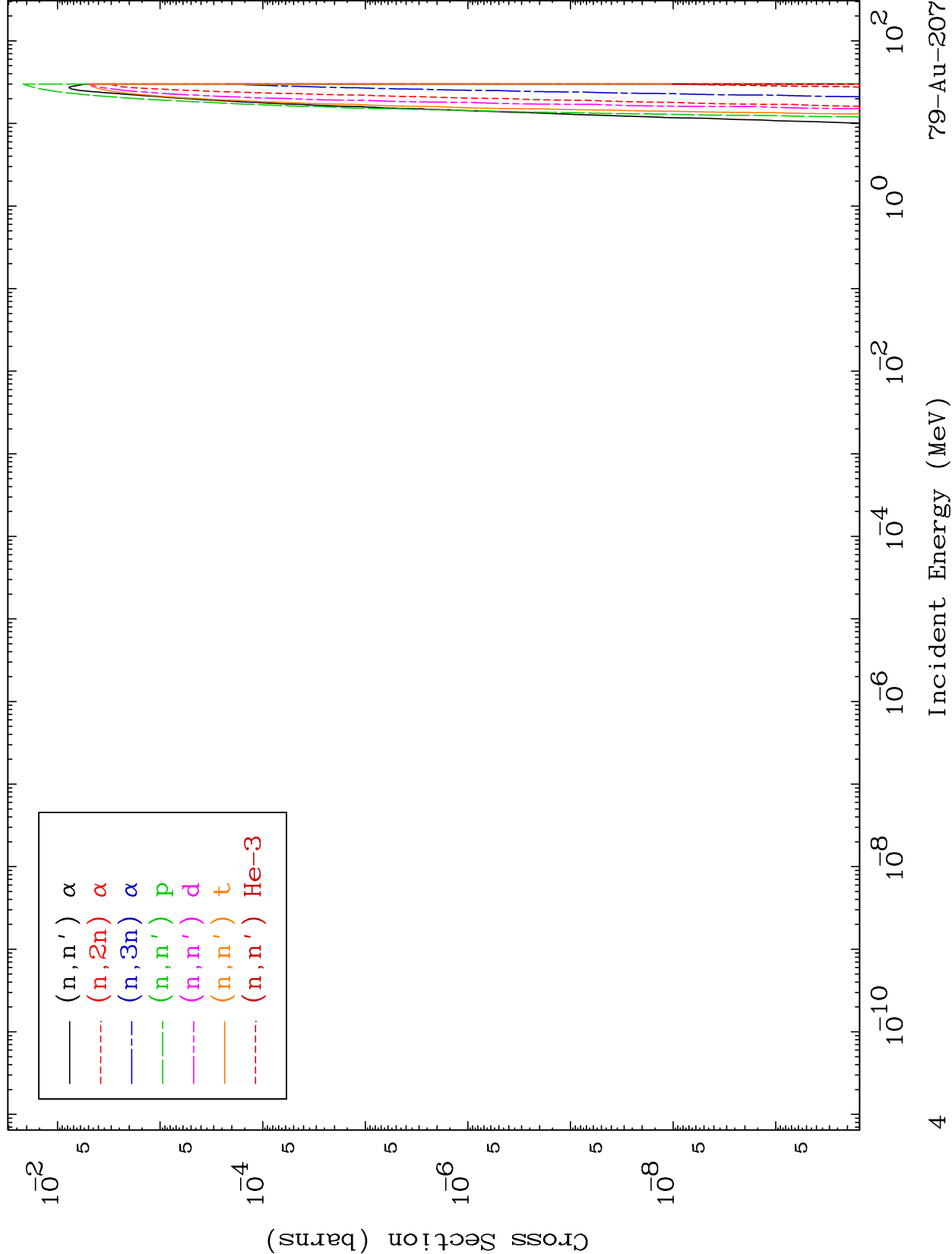
MAT 7955

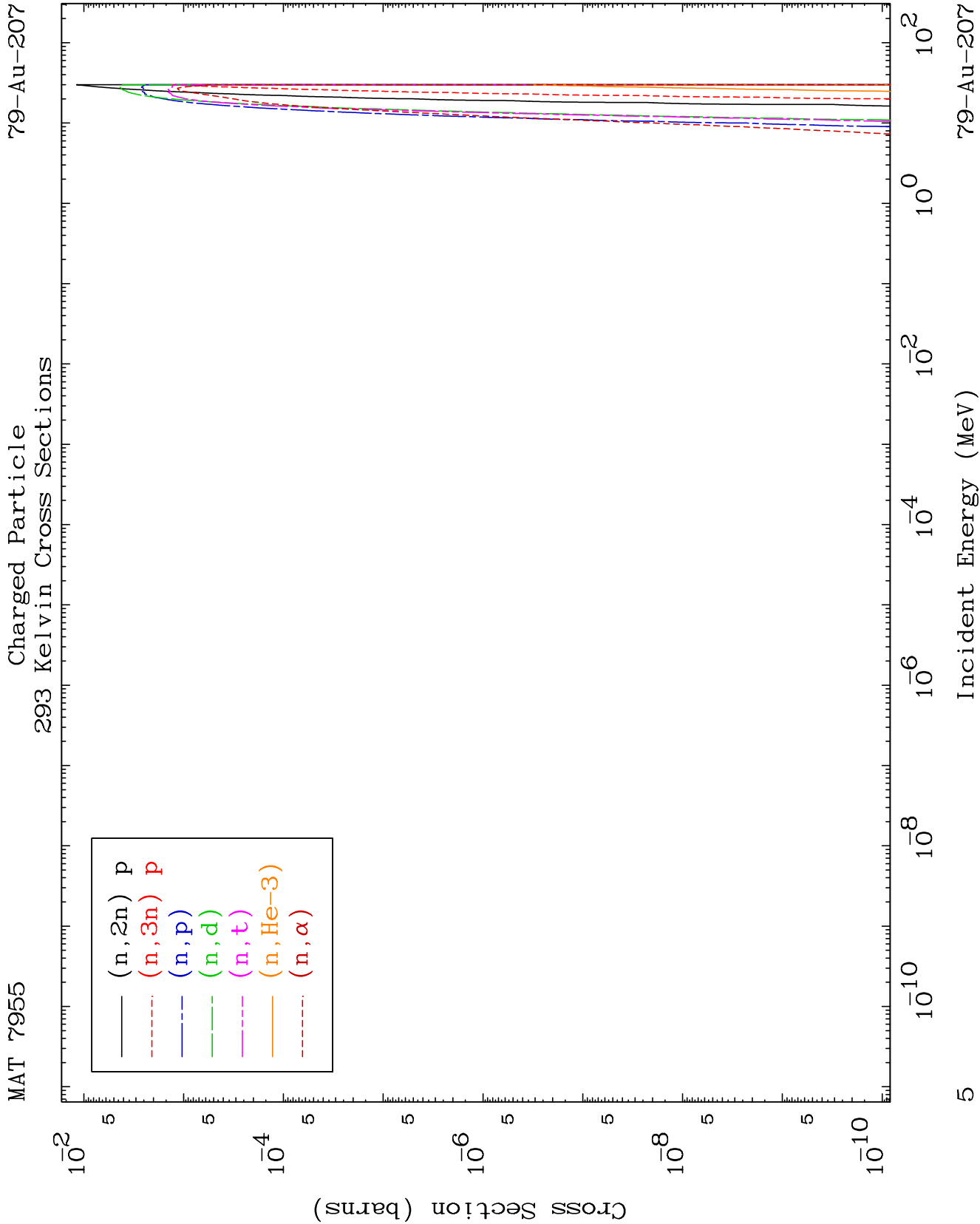
Neutron Production
293 Kelvin Cross Sections

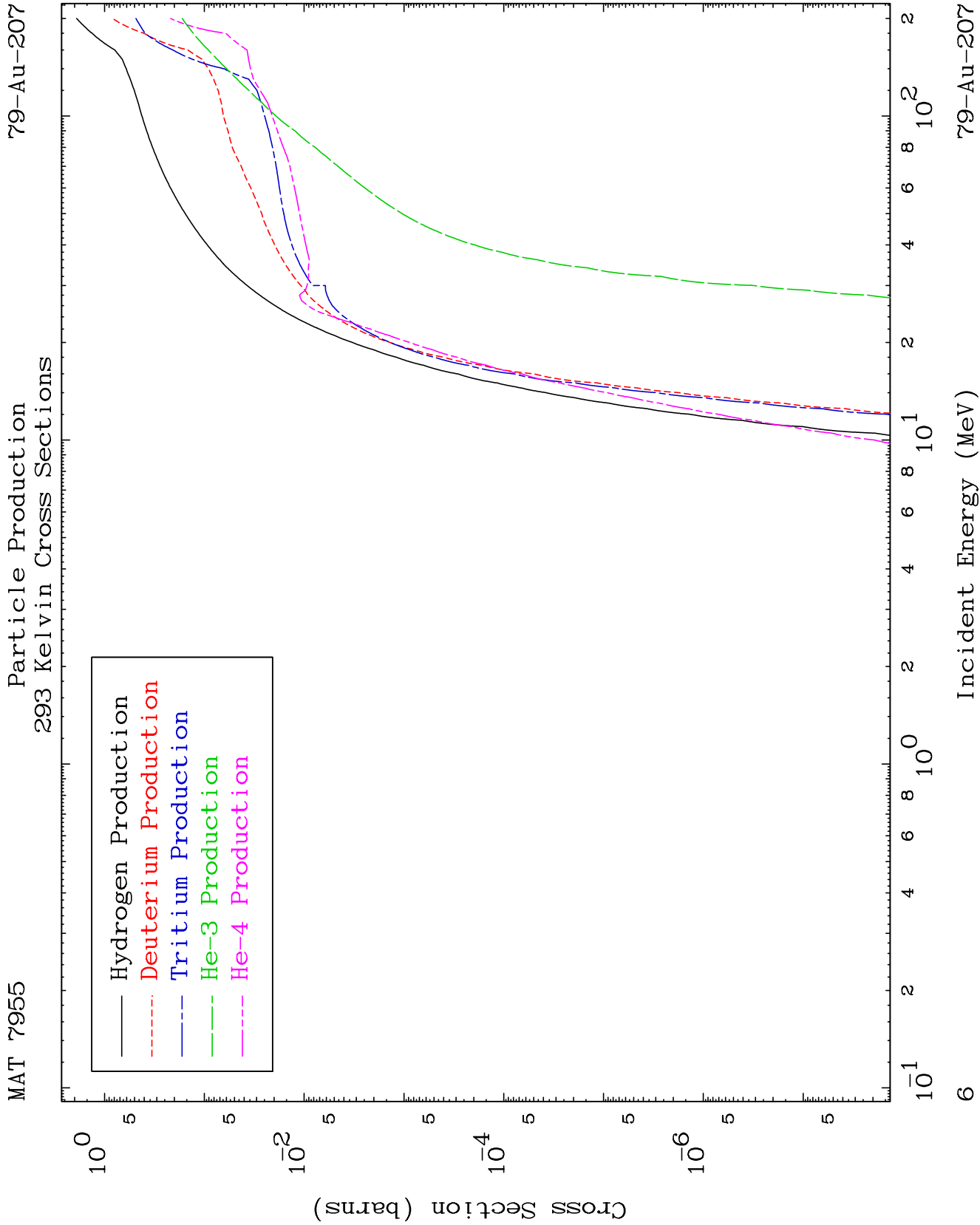
79-Au-207

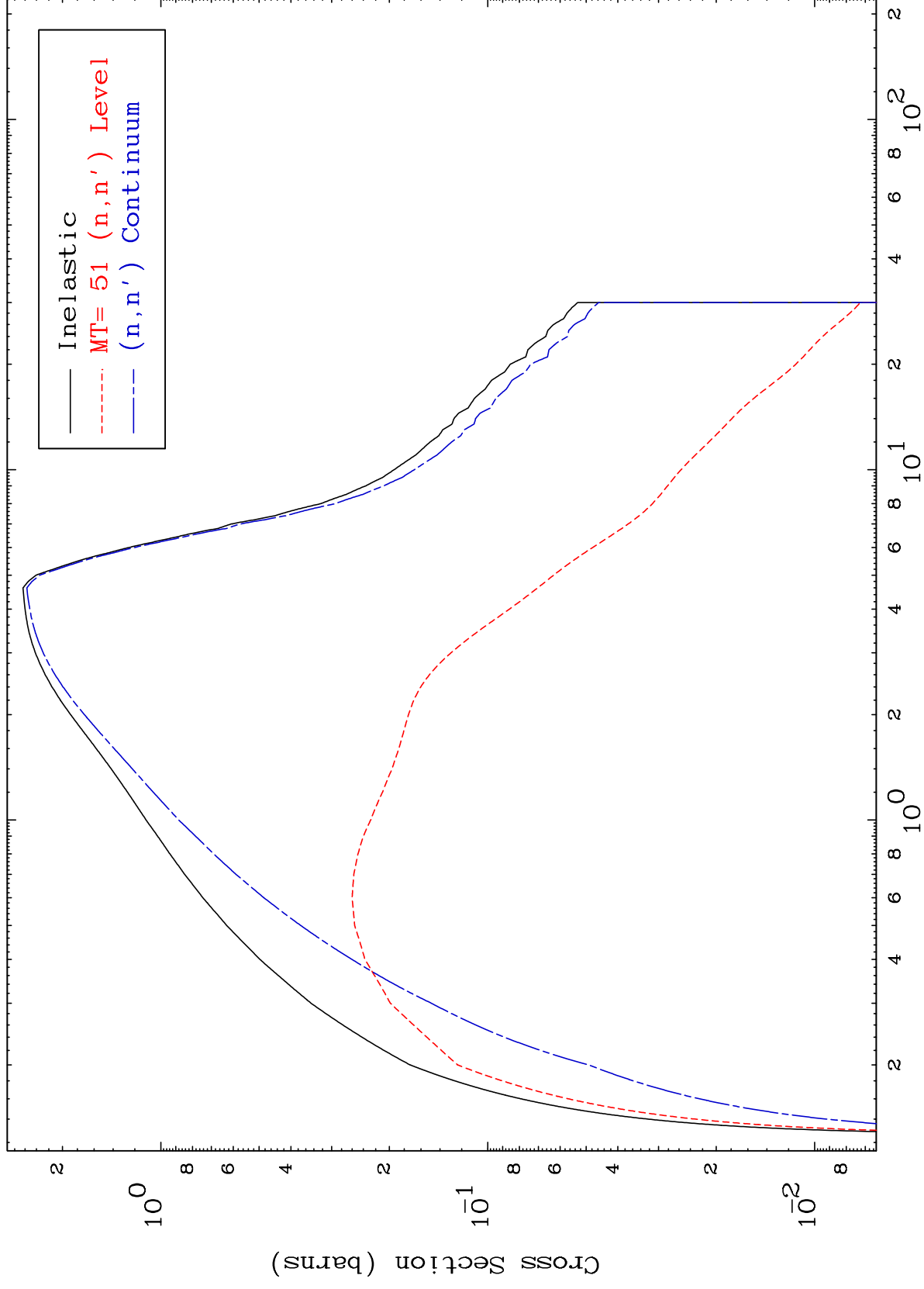


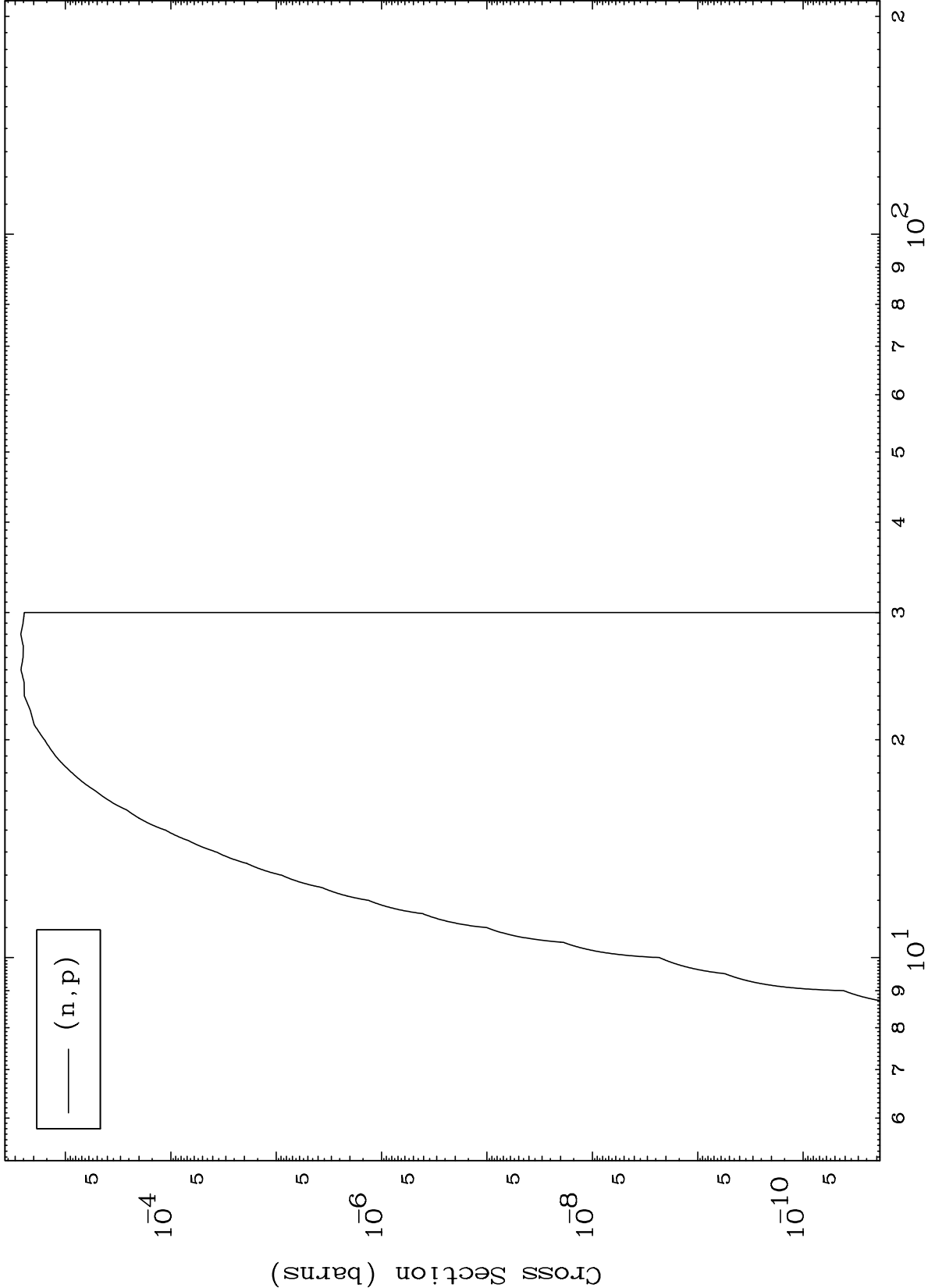


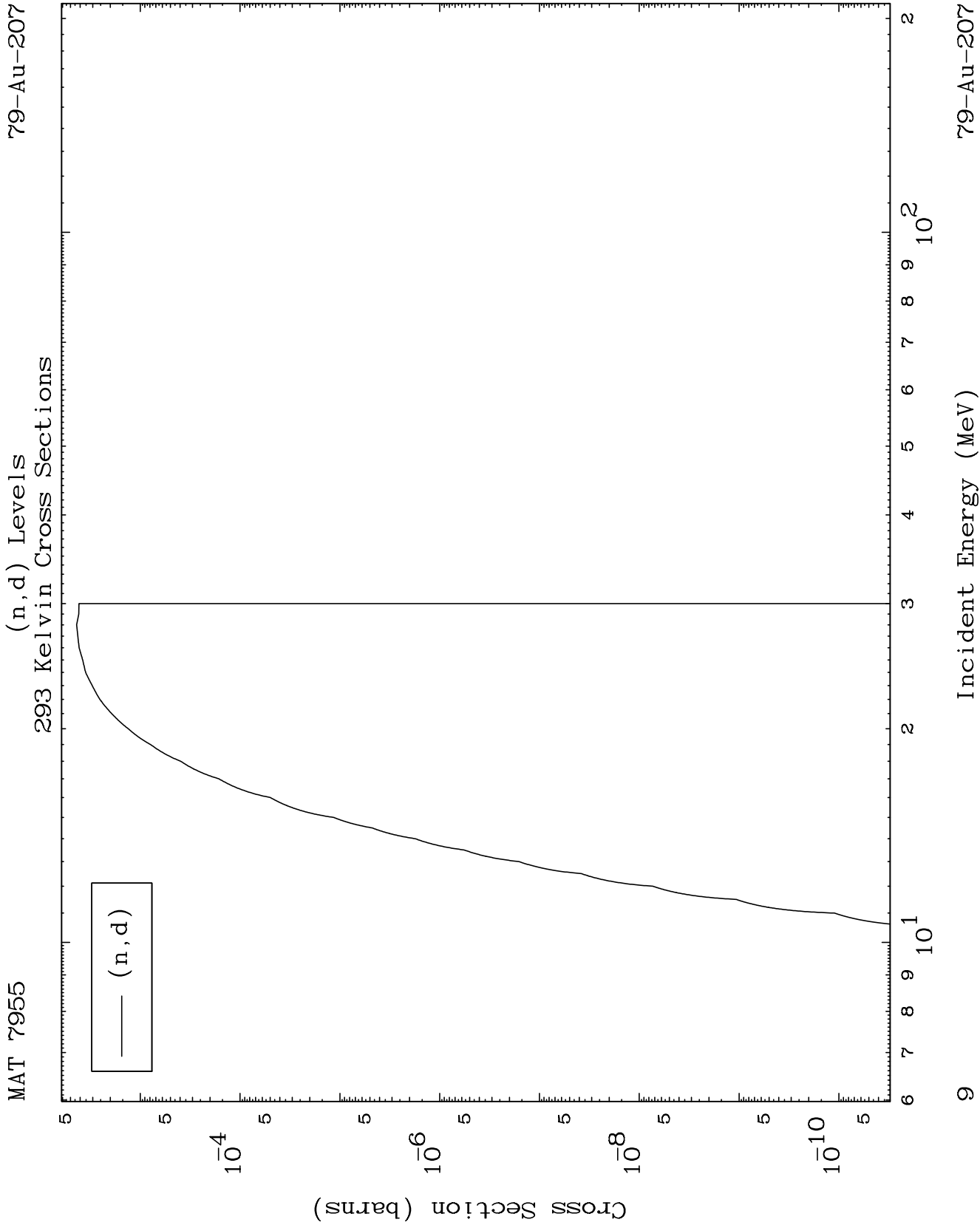








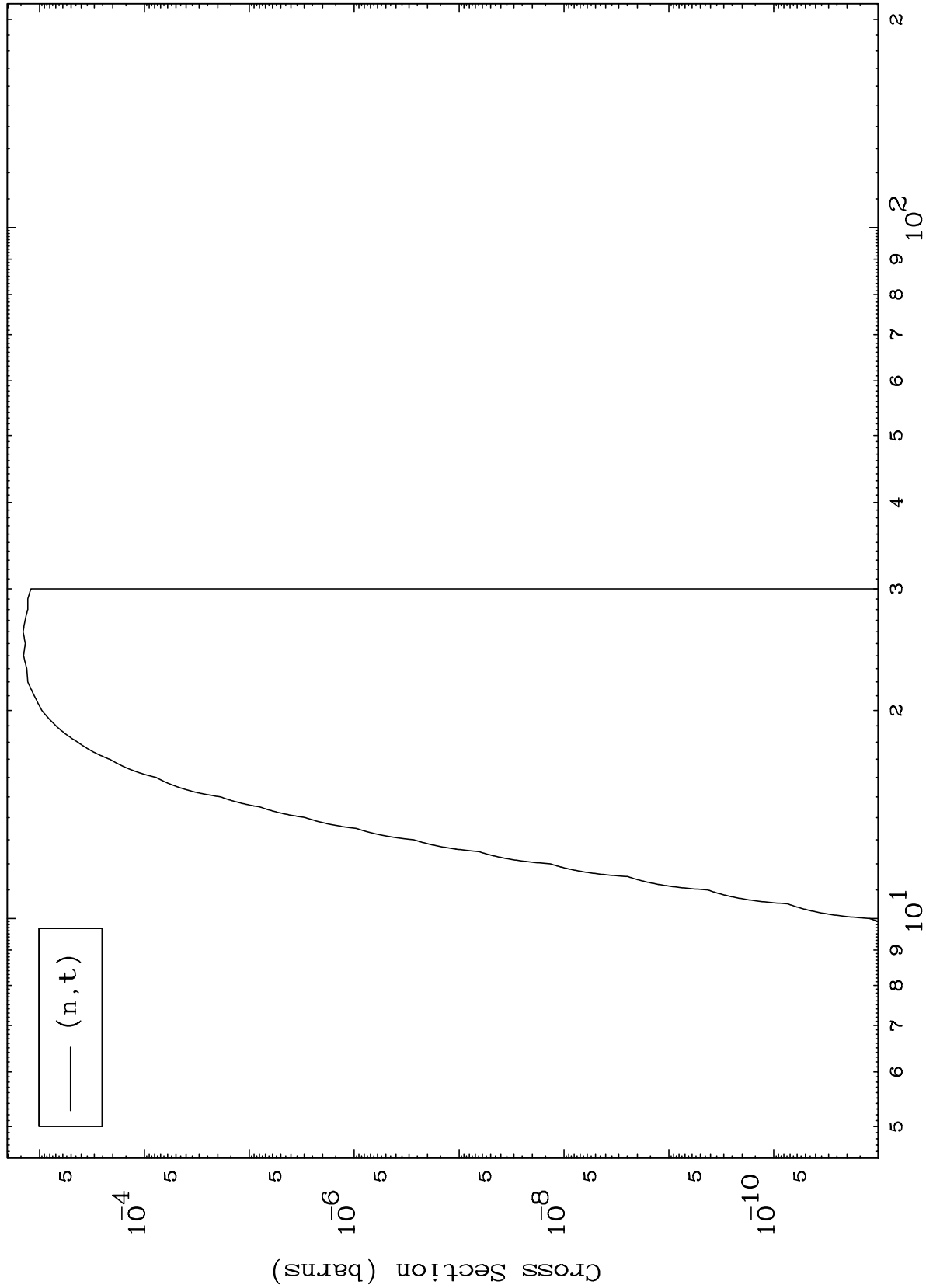




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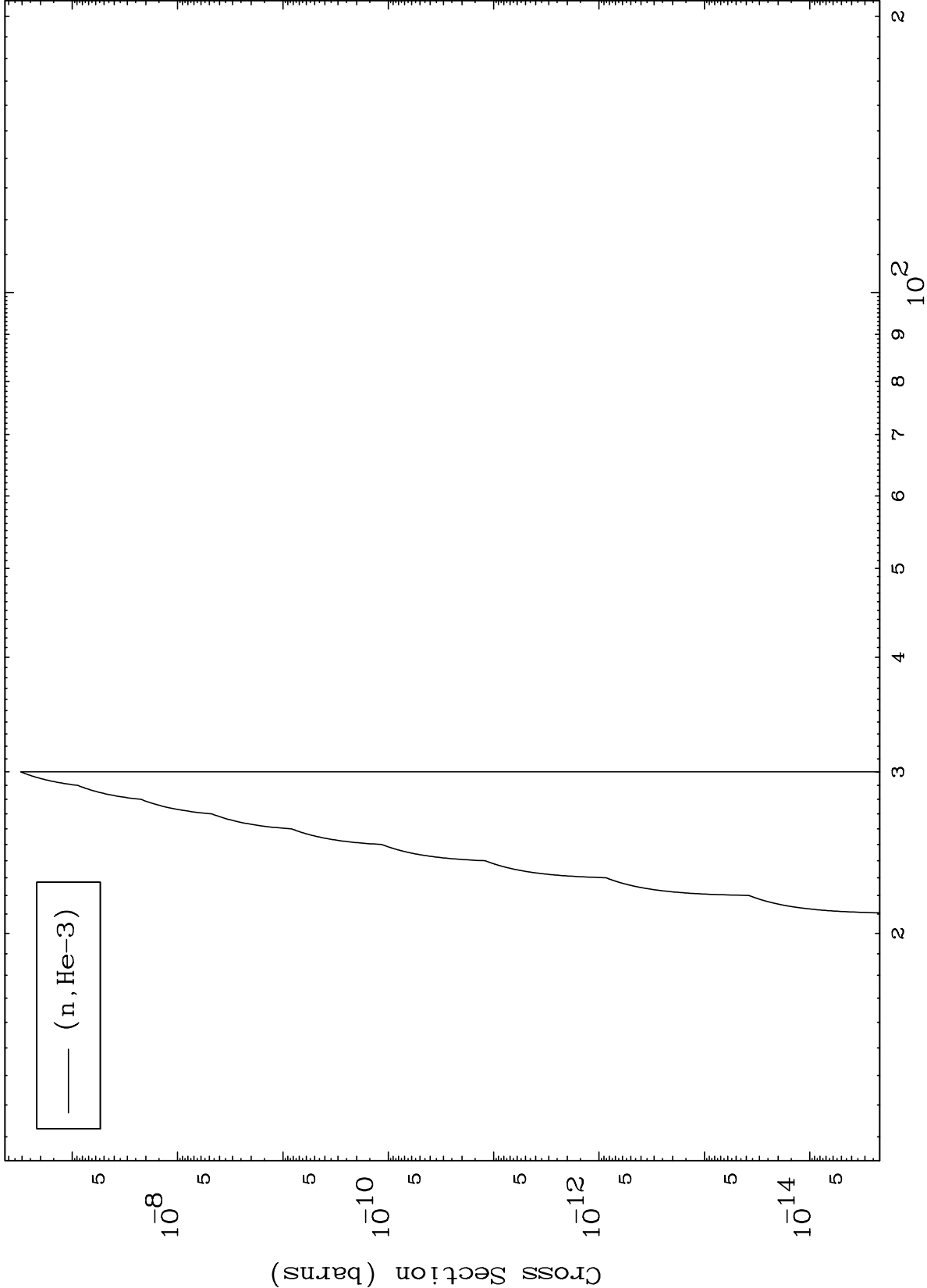
(n,t) Levels
293 Kelvin Cross Sections



79-Au-207

Incident Energy (MeV)

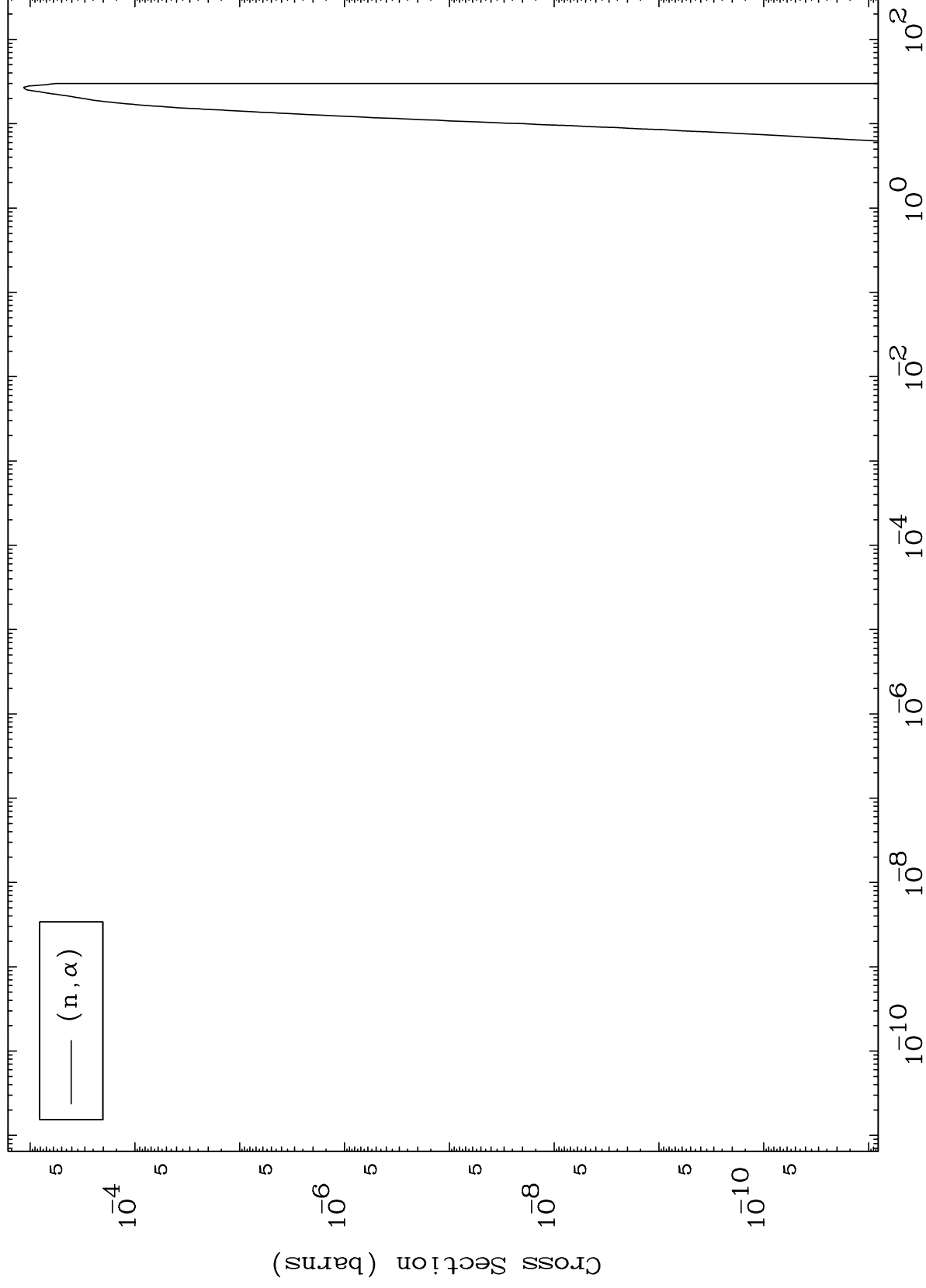
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(n, α) Levels
293 Kelvin Cross Sections

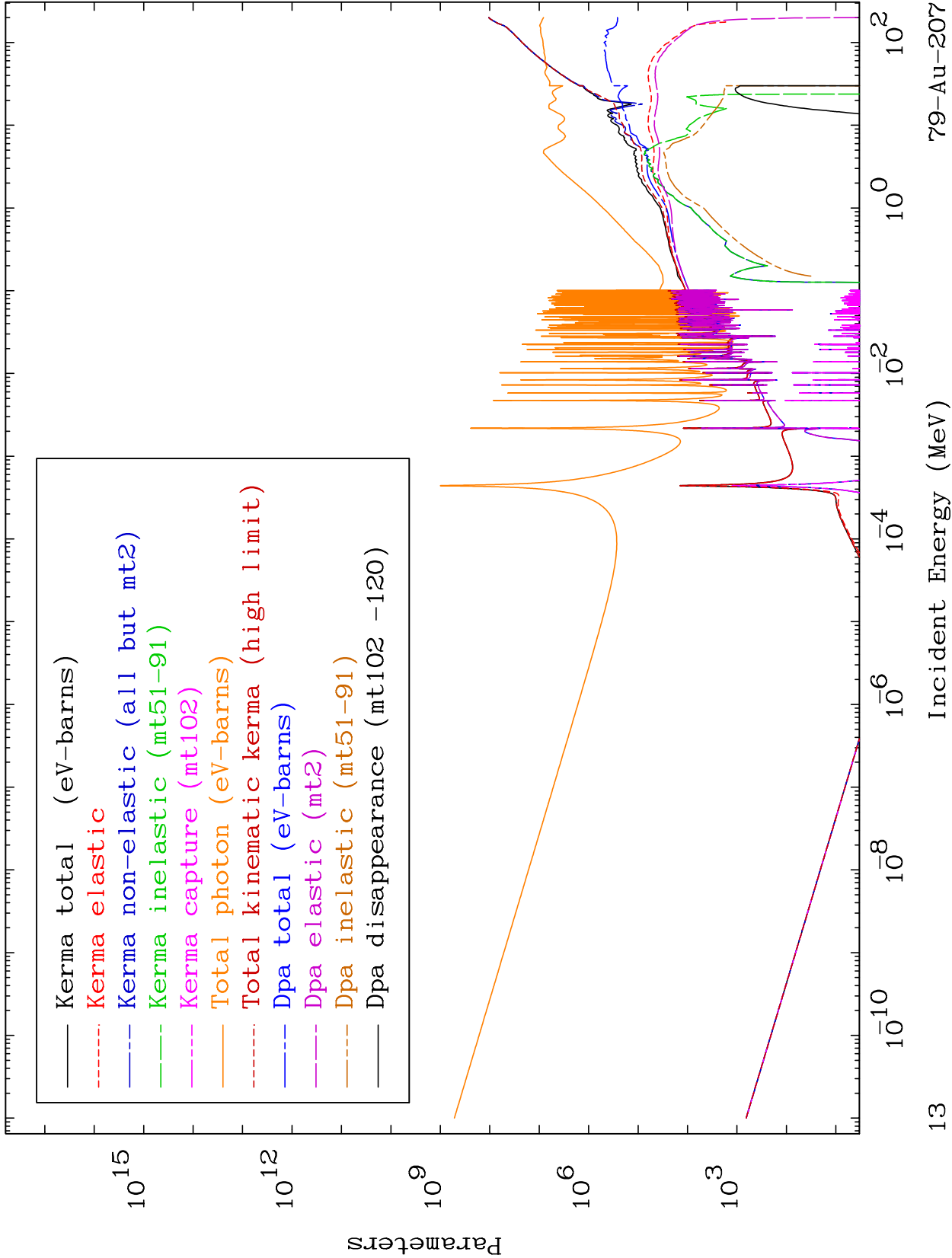
79-Au-207

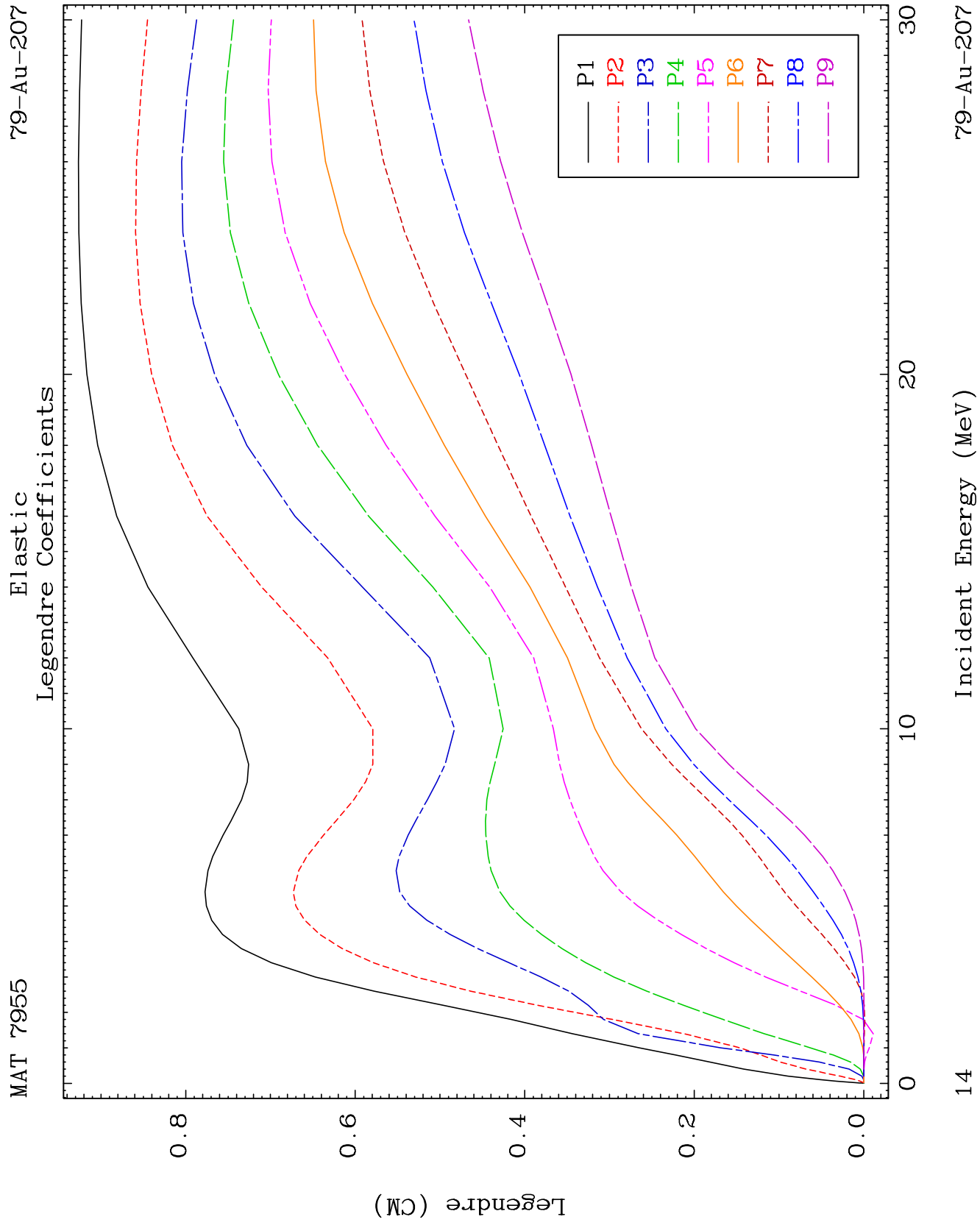


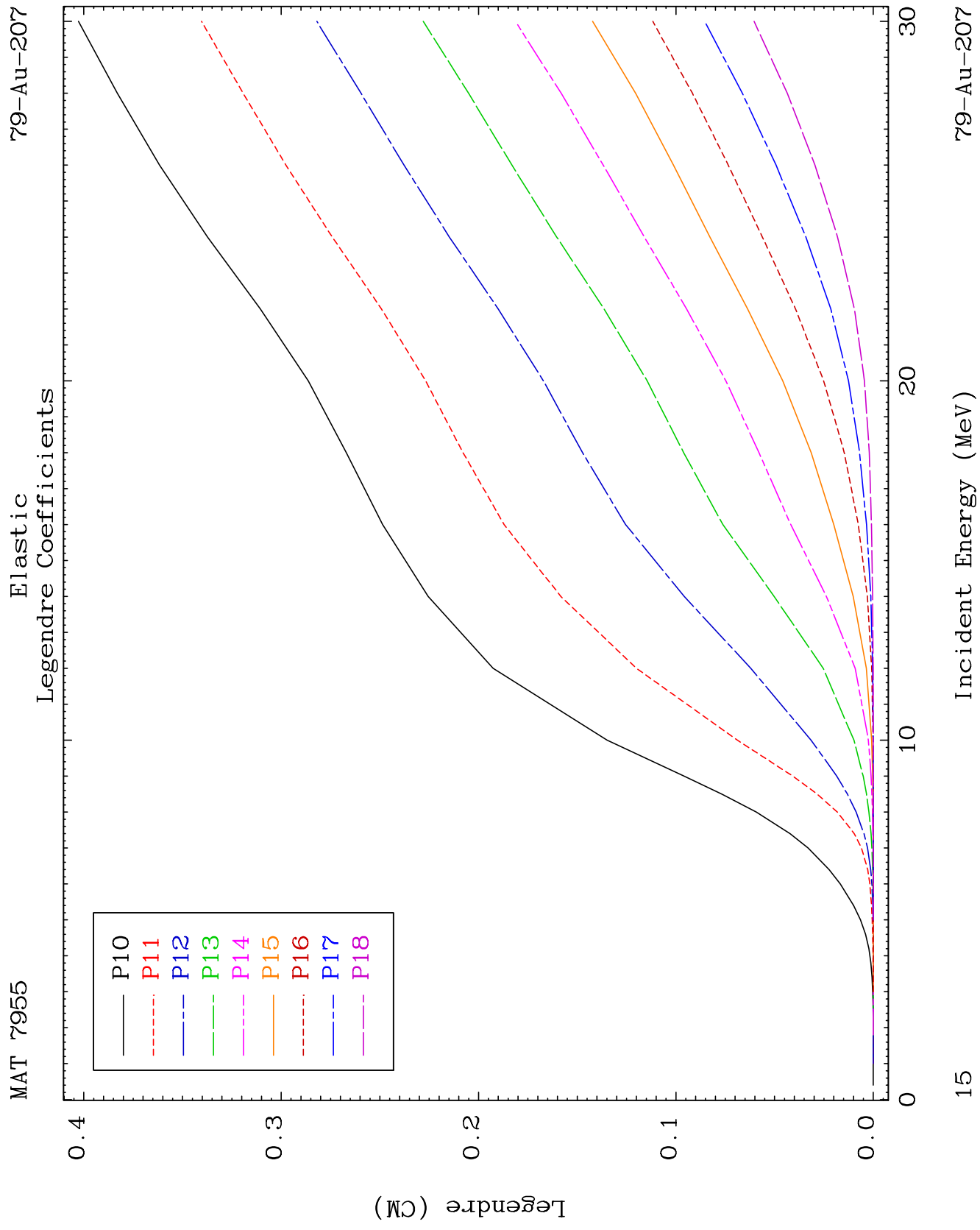
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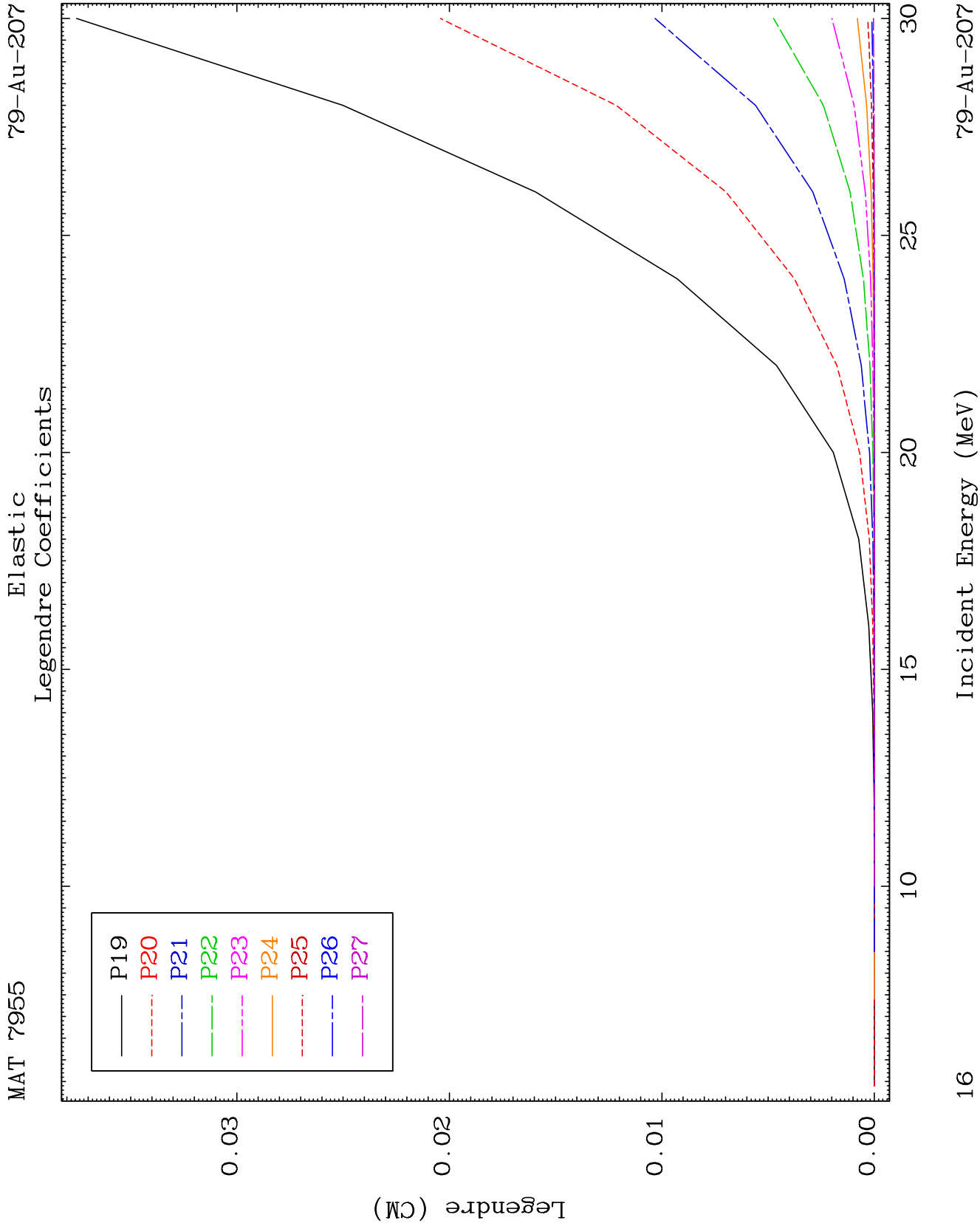
Incident Energy (MeV)

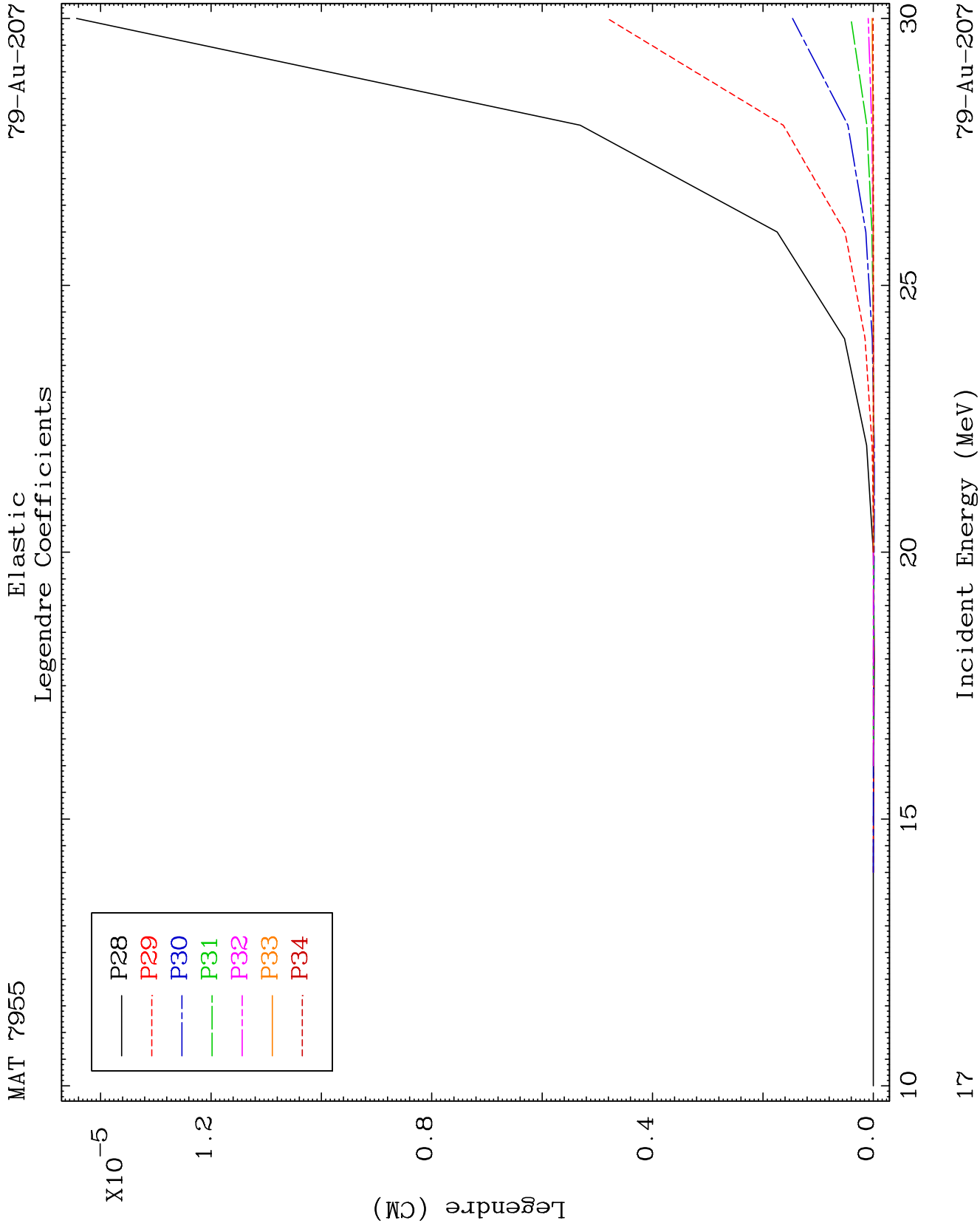
79-Au-207

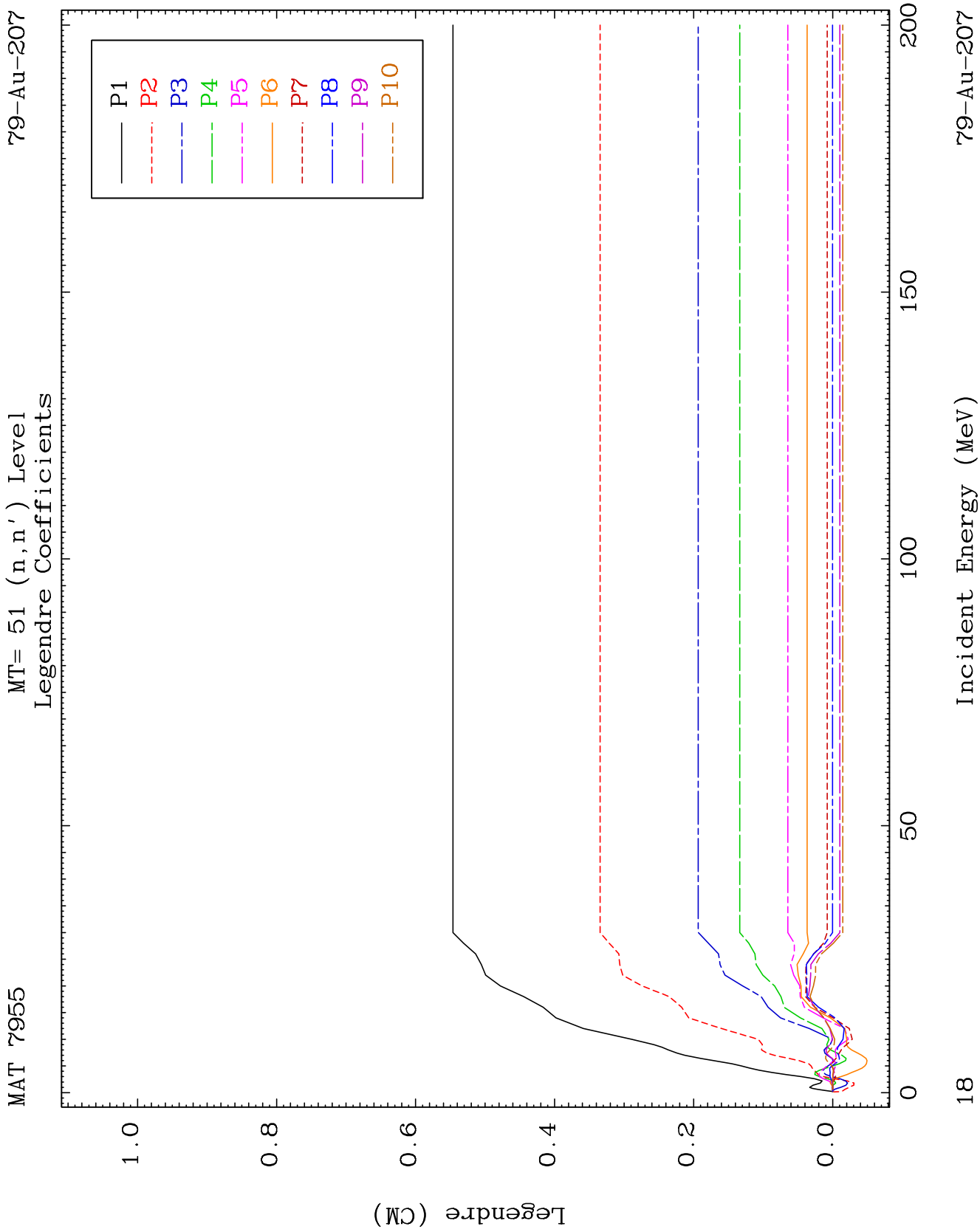


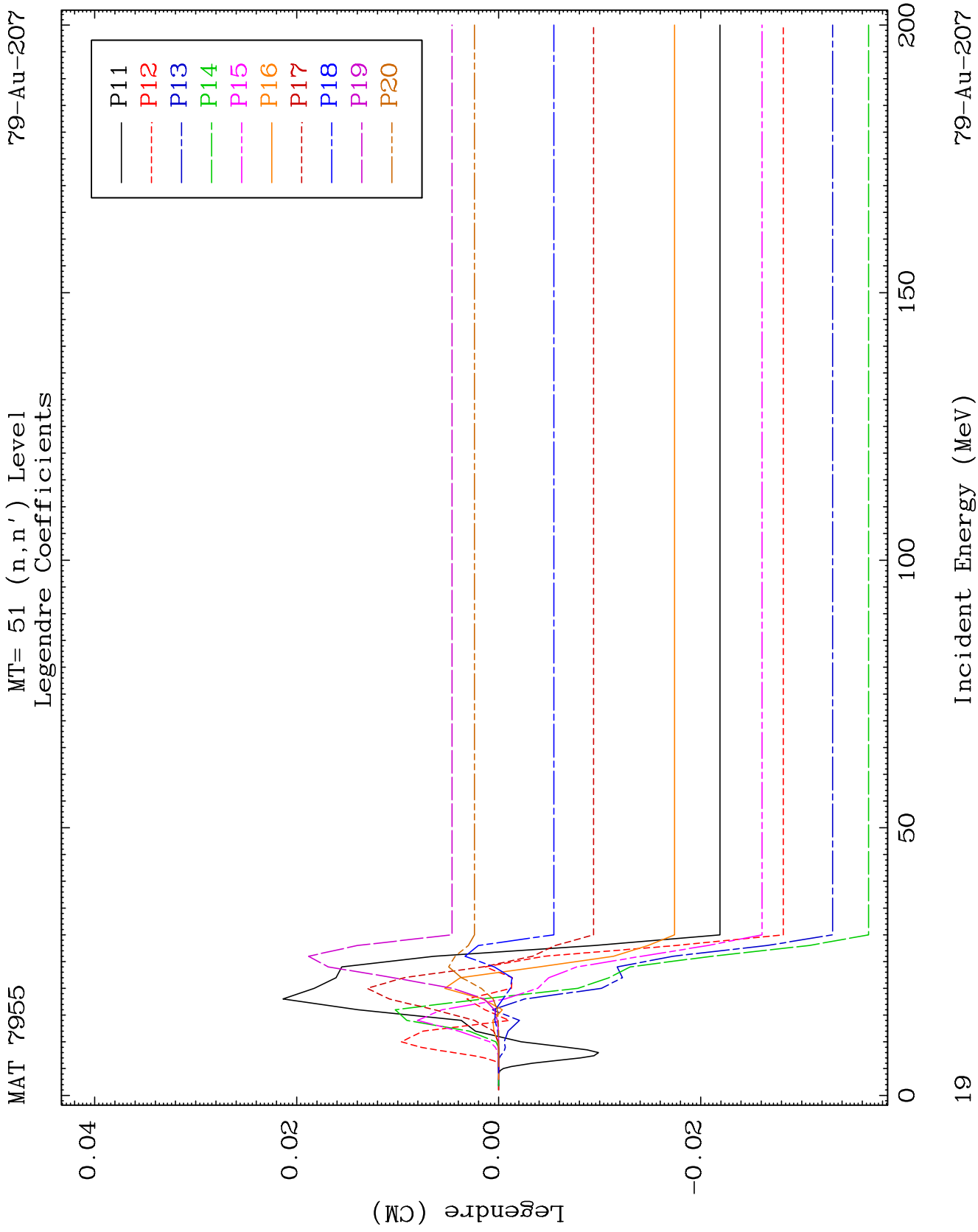








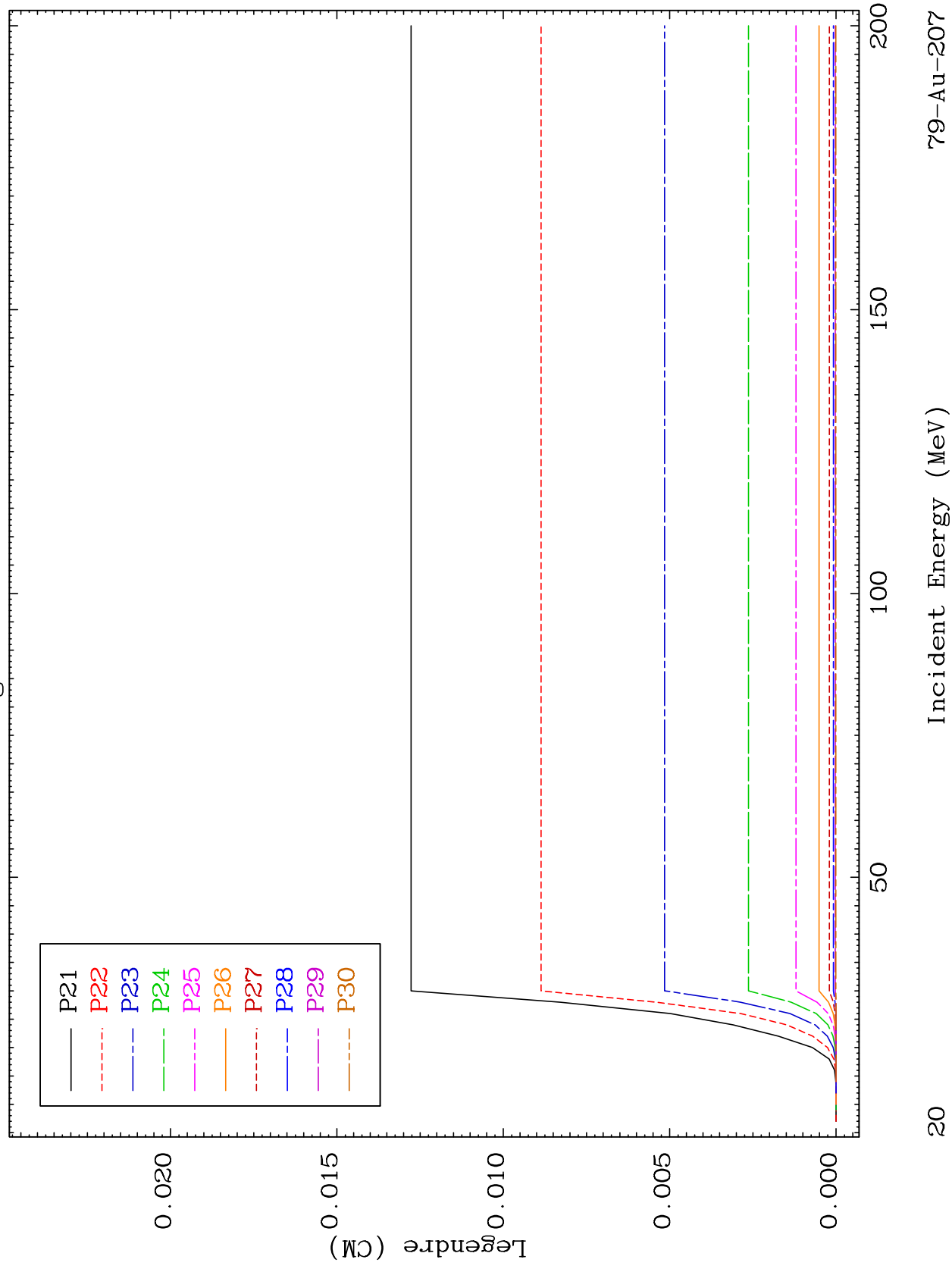


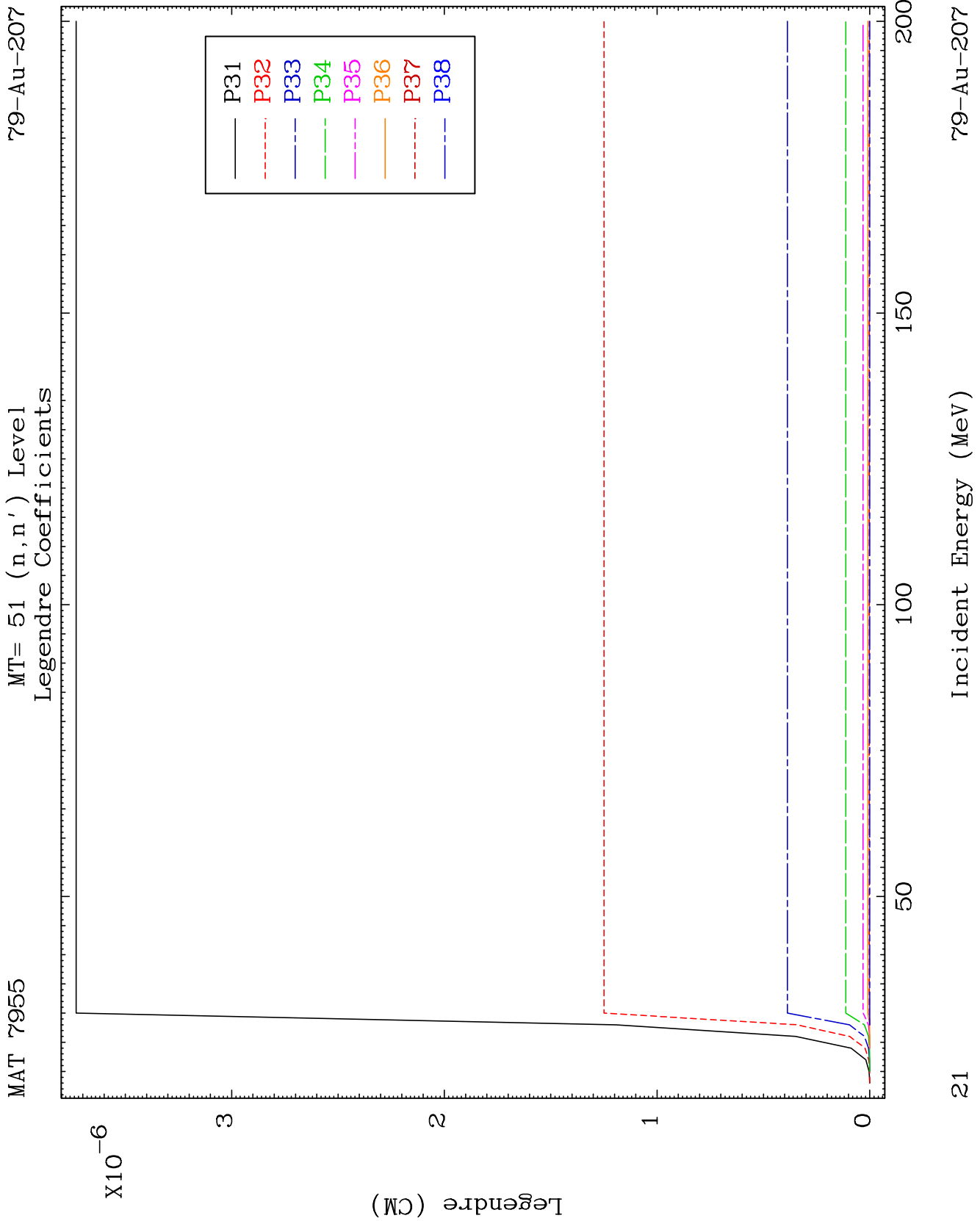


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MT= 51 (n, n') Level
Legendre Coefficients

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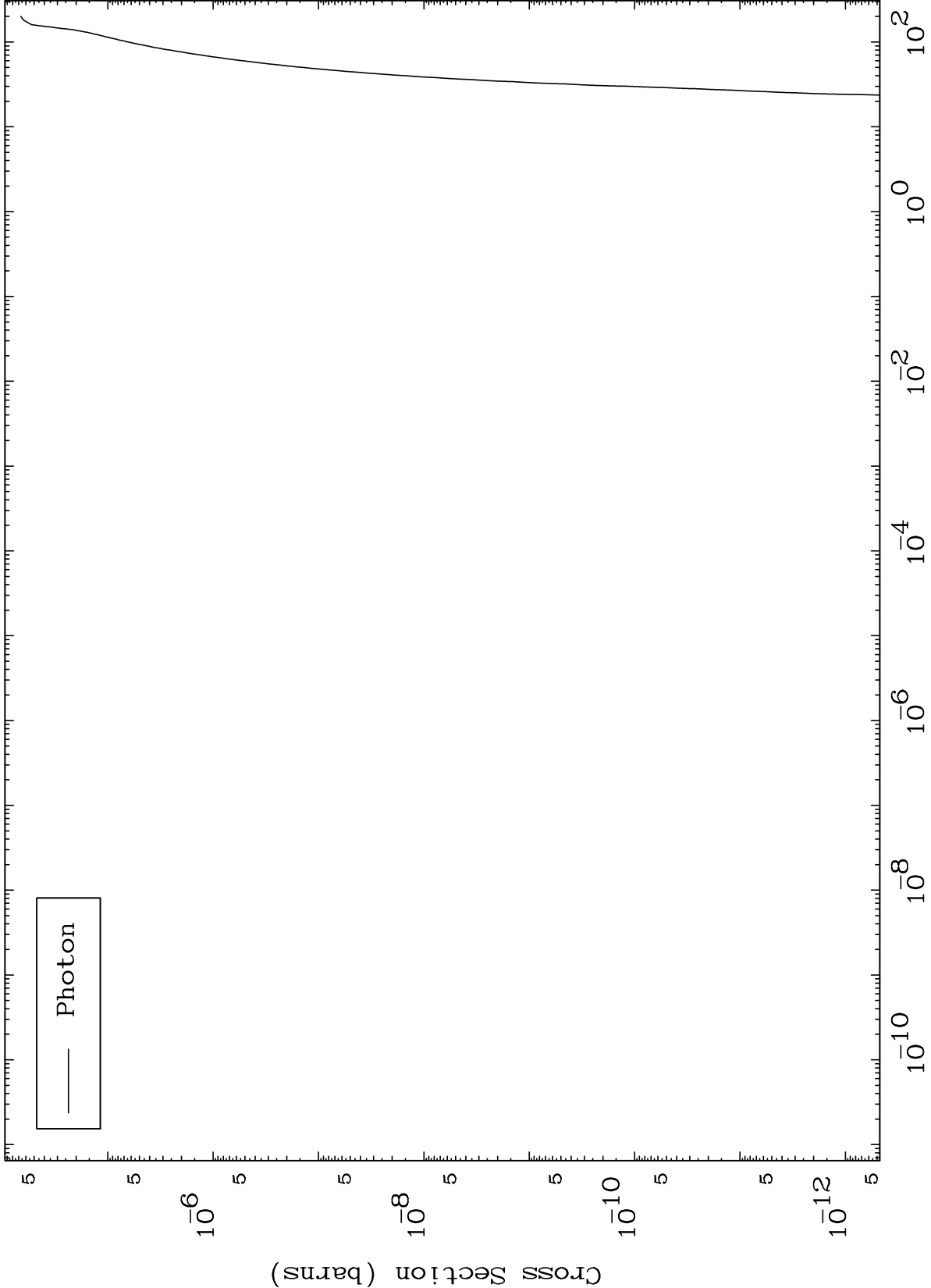


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Fission

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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