

Program EVALPLOT  
(Version 2017-1)

by

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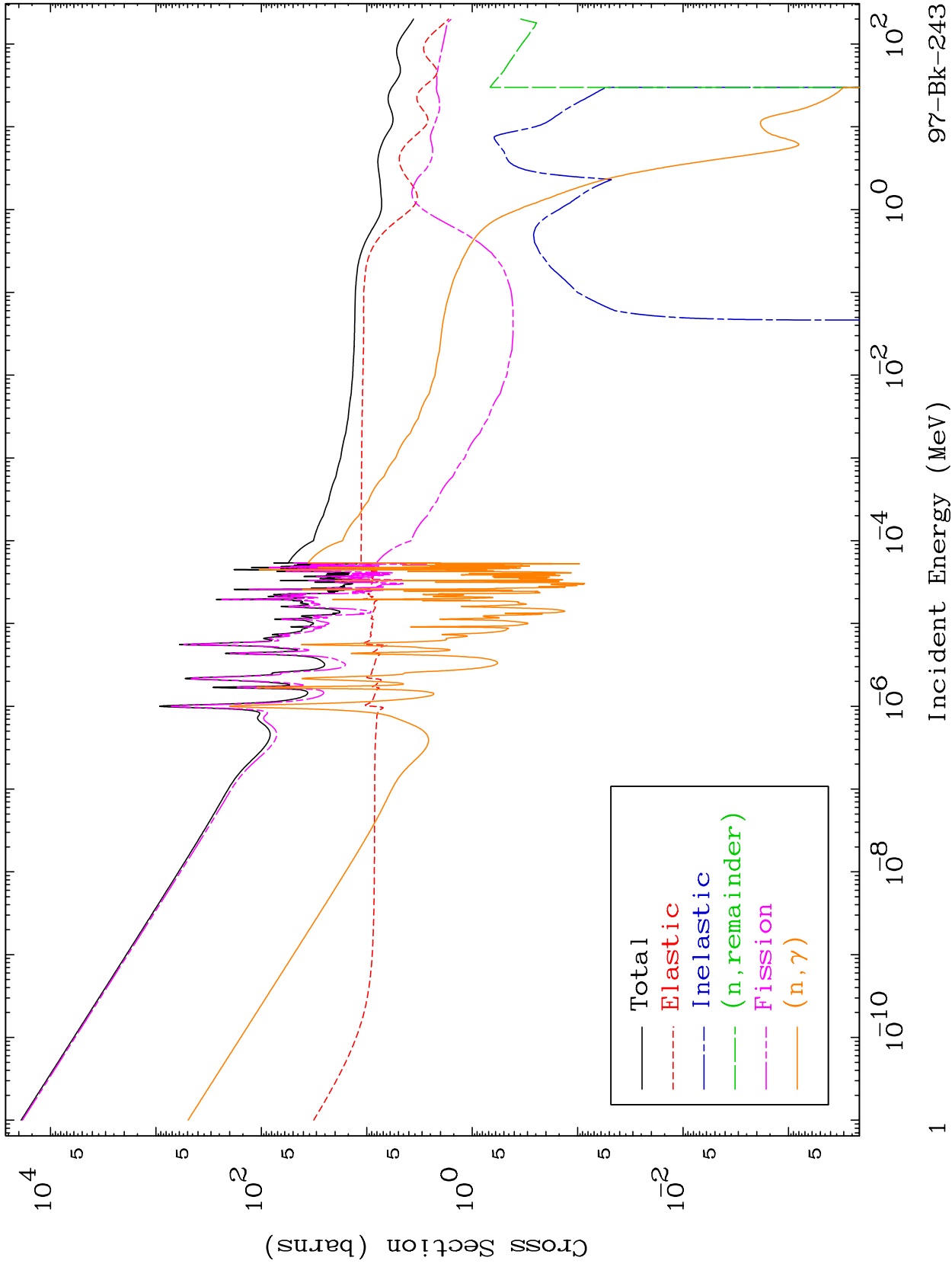
Press Mouse Button to Start

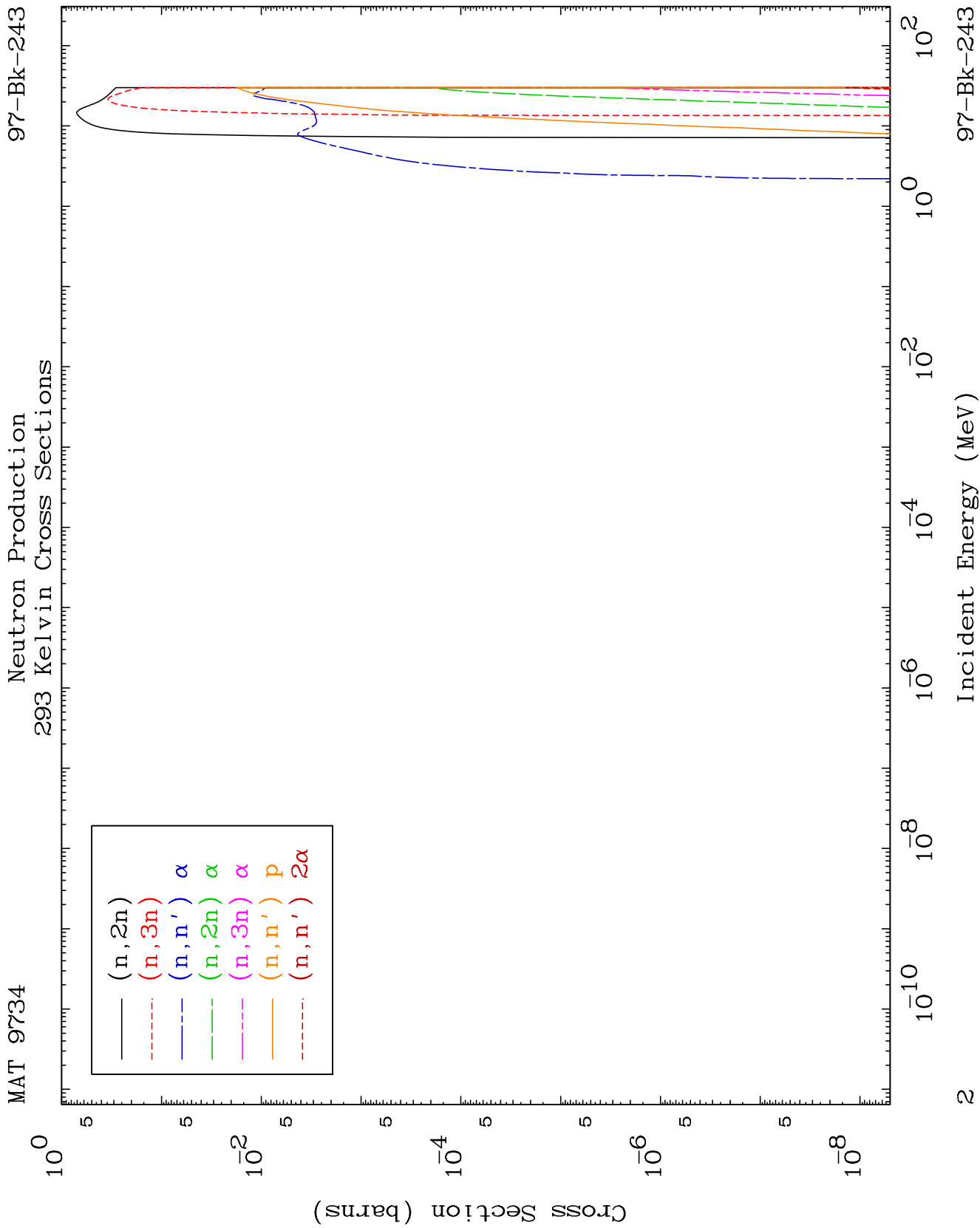
MAT 9734

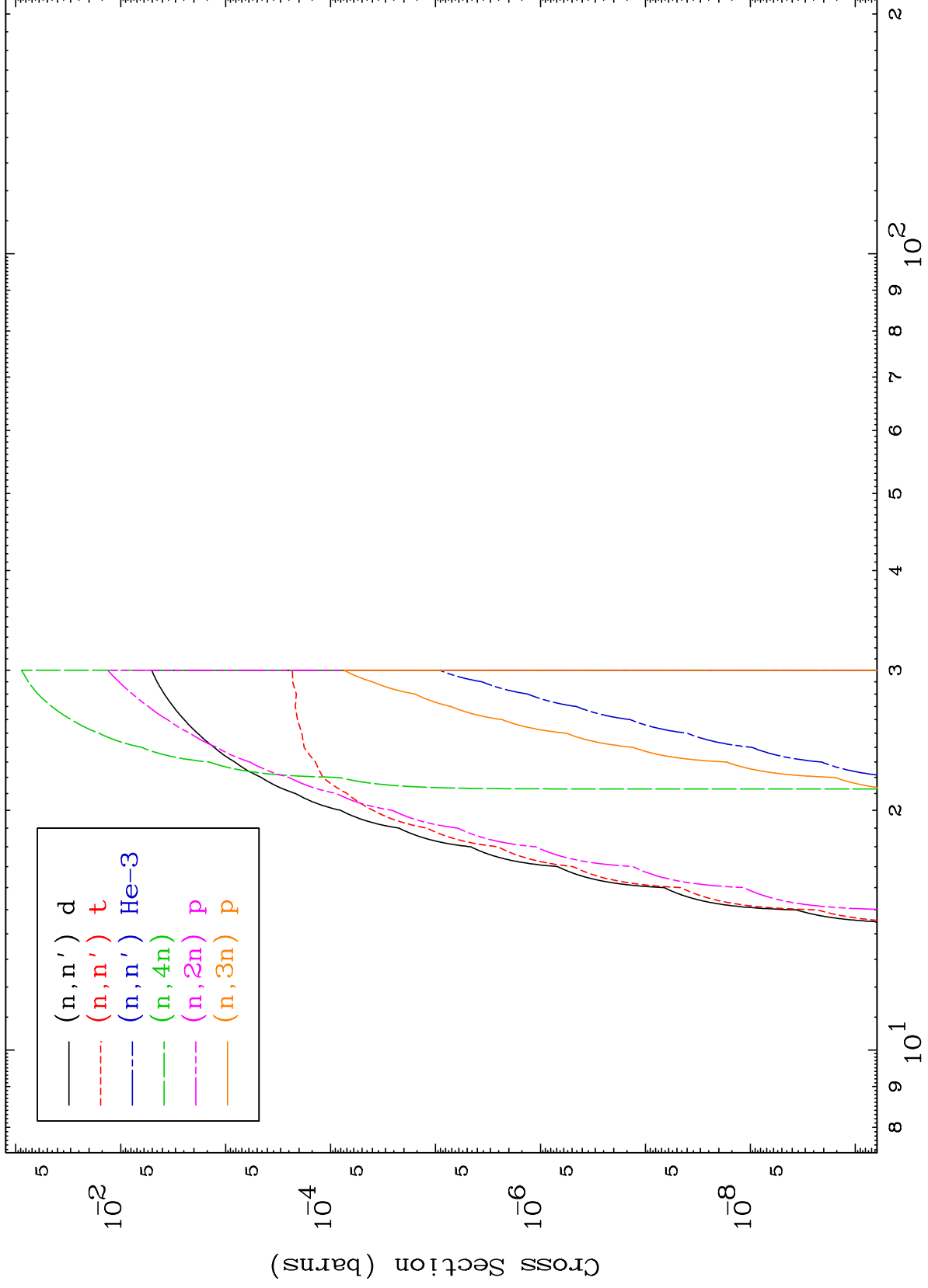
Major

293 Kelvin Cross Sections

97-Bk-243



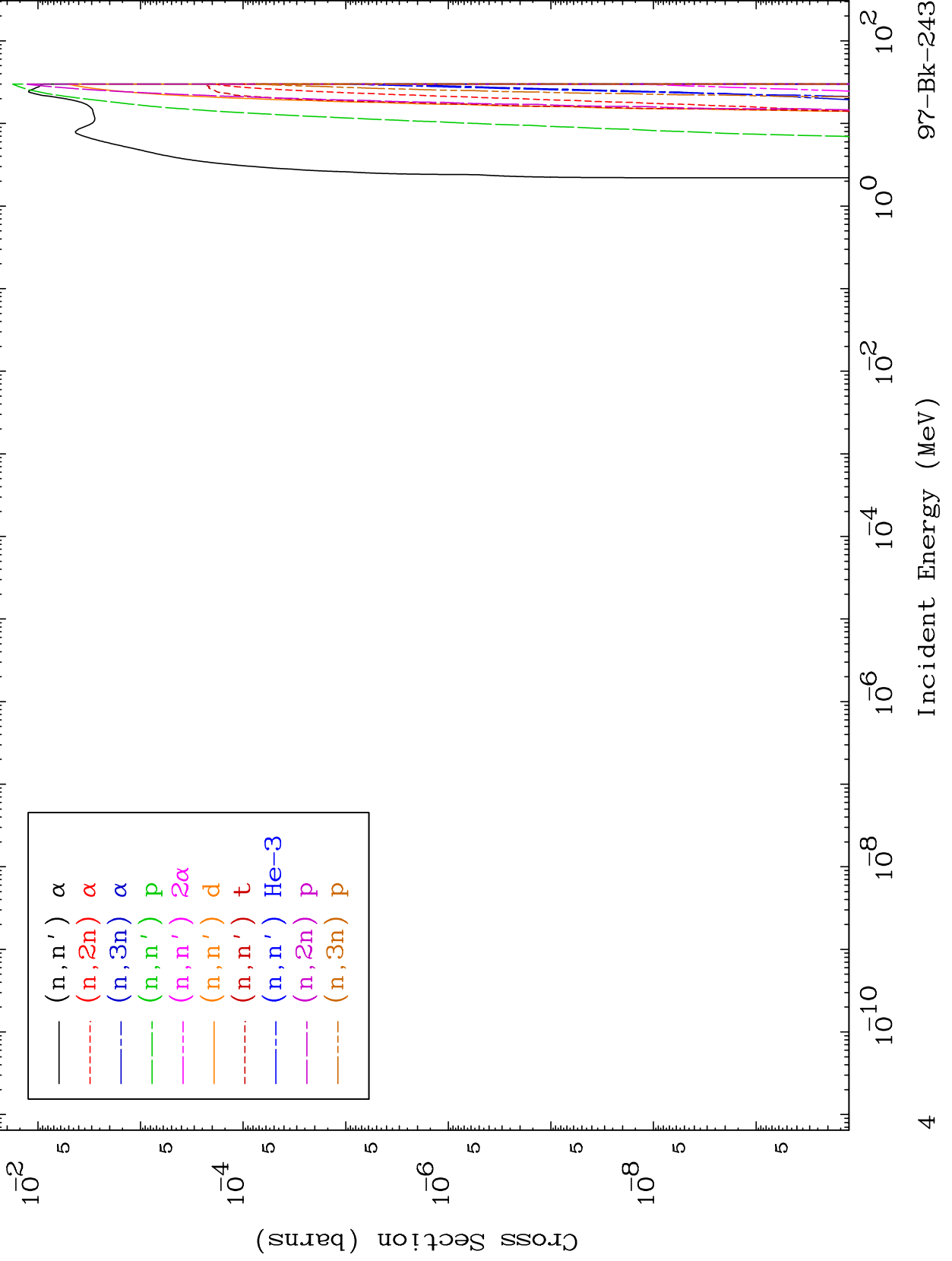




MAT 9734

Charged Particle  
293 Kelvin Cross Sections

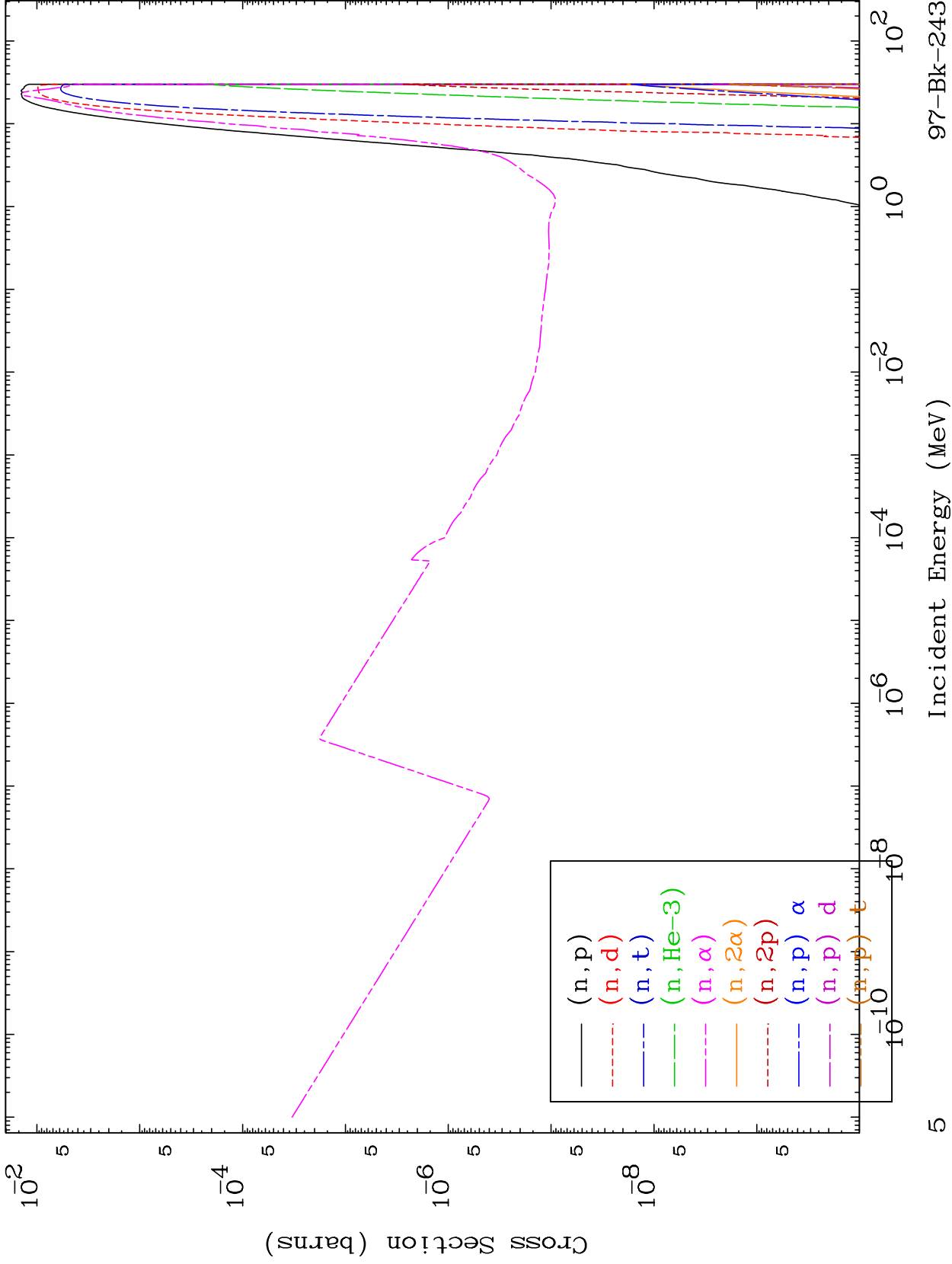
97-Bk-243



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Charged Particle  
293 Kelvin Cross Sections

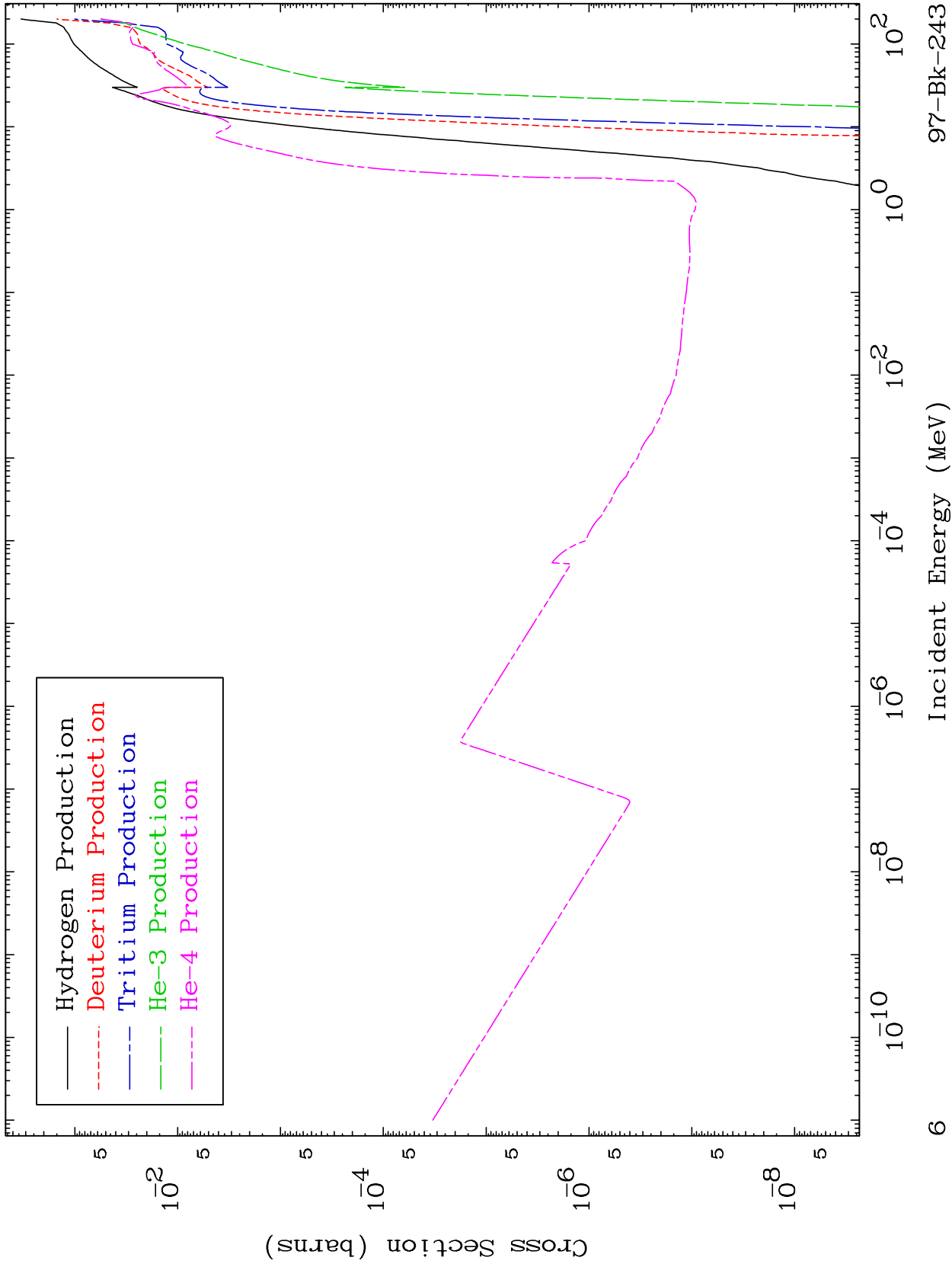
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Particle Production  
293 Kelvin Cross Sections

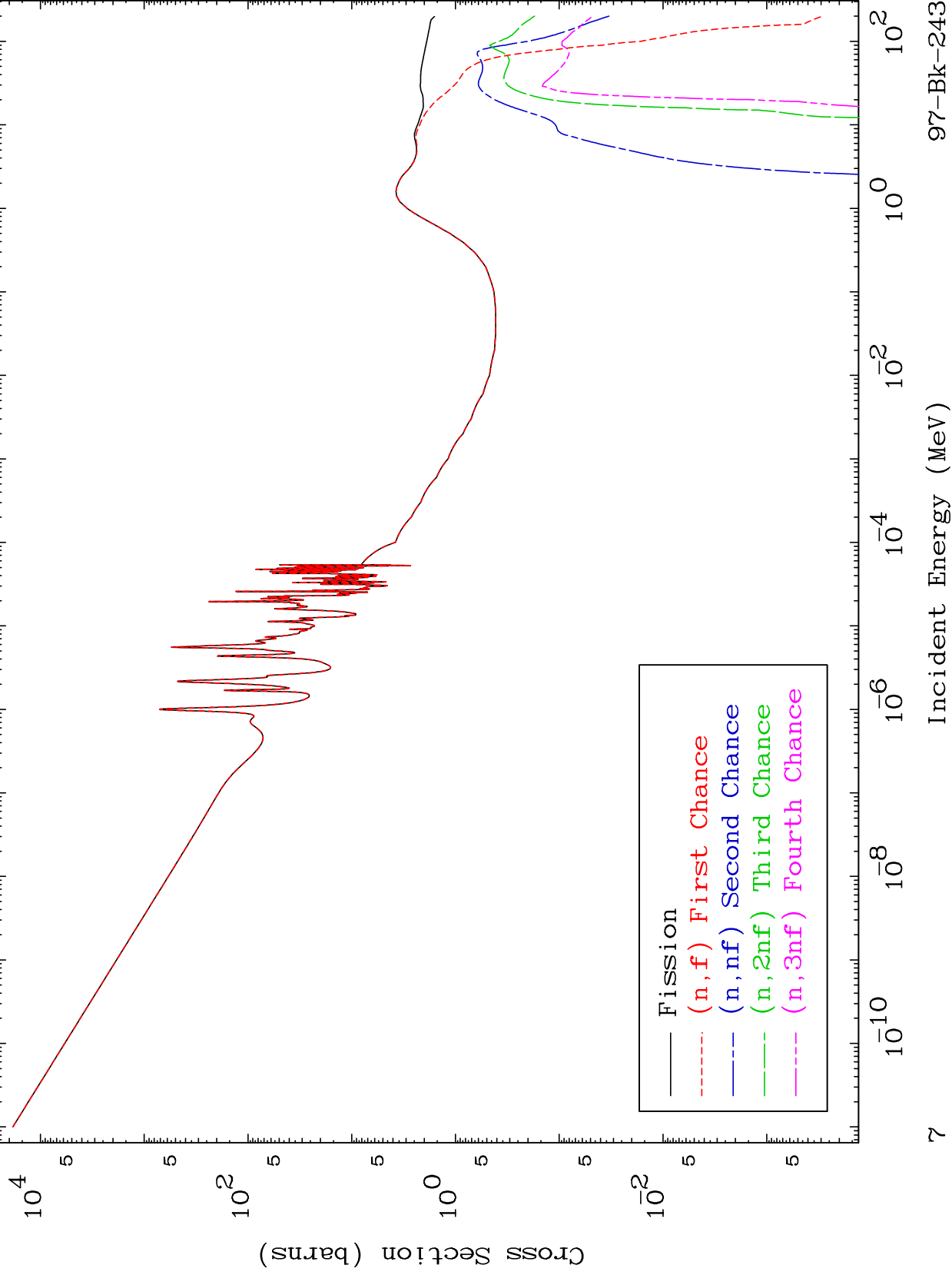
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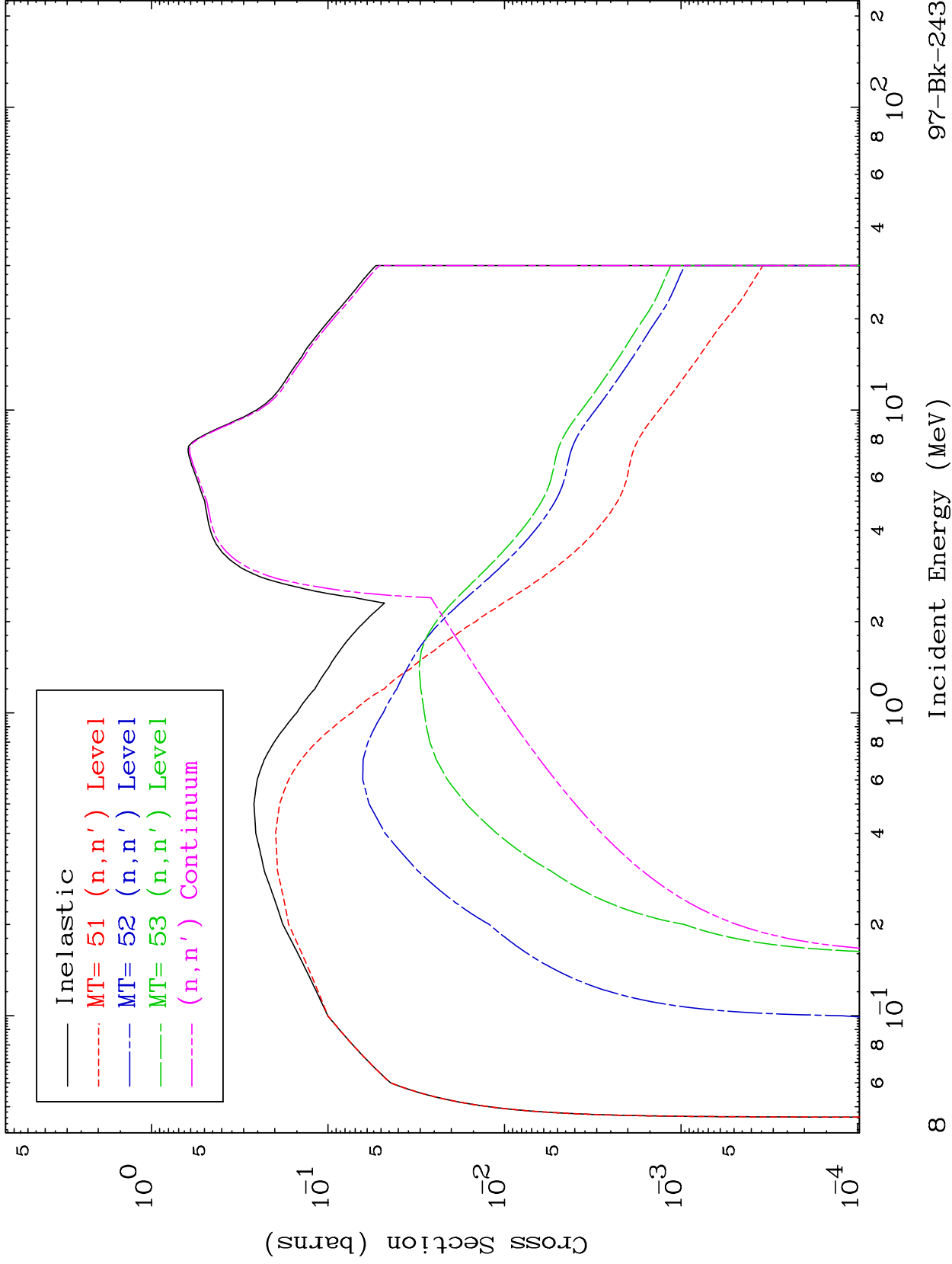


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Fission  
293 Kelvin Cross Sections

97-Bk-243

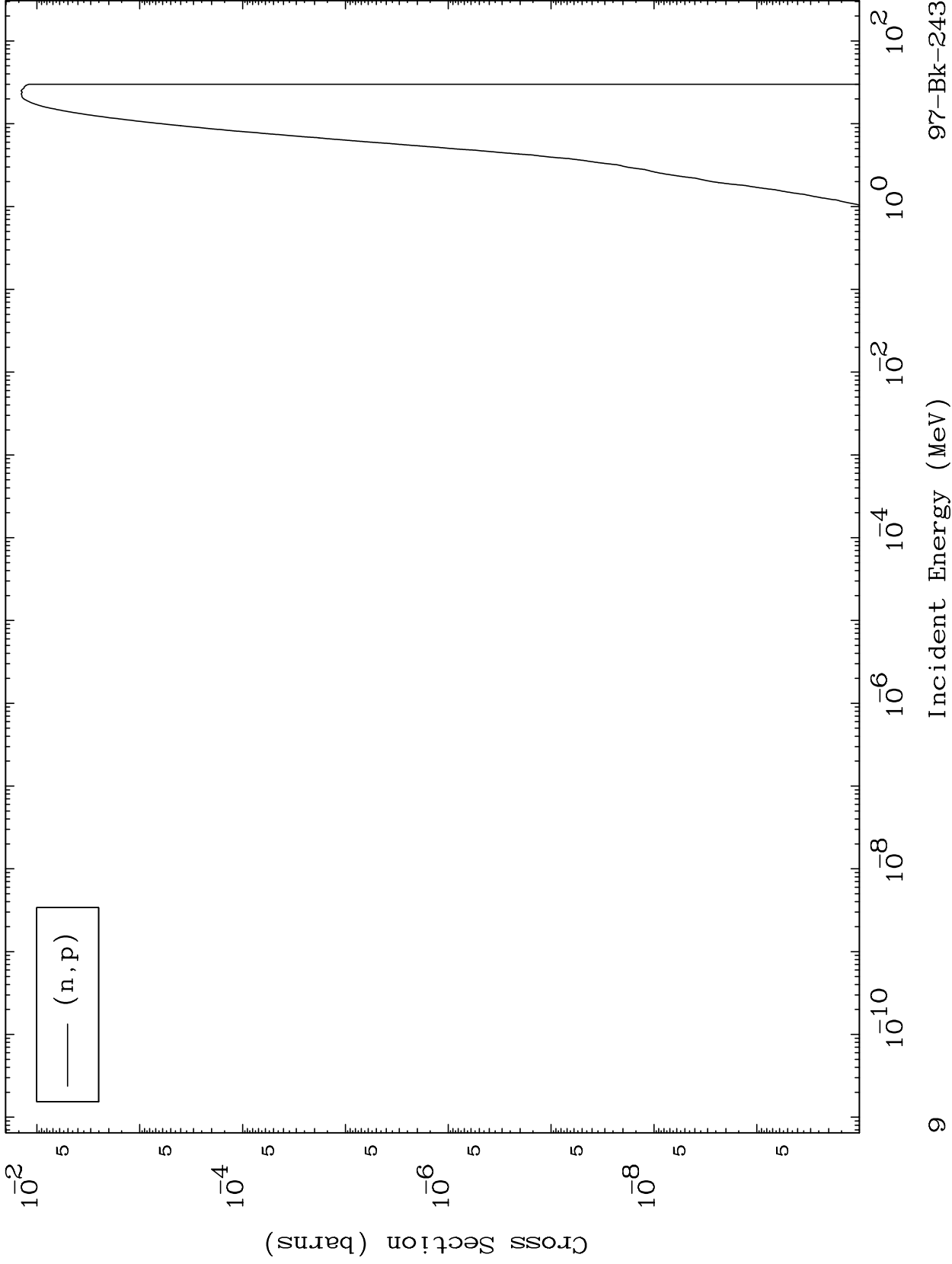




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(n,p) Levels  
293 Kelvin Cross Sections

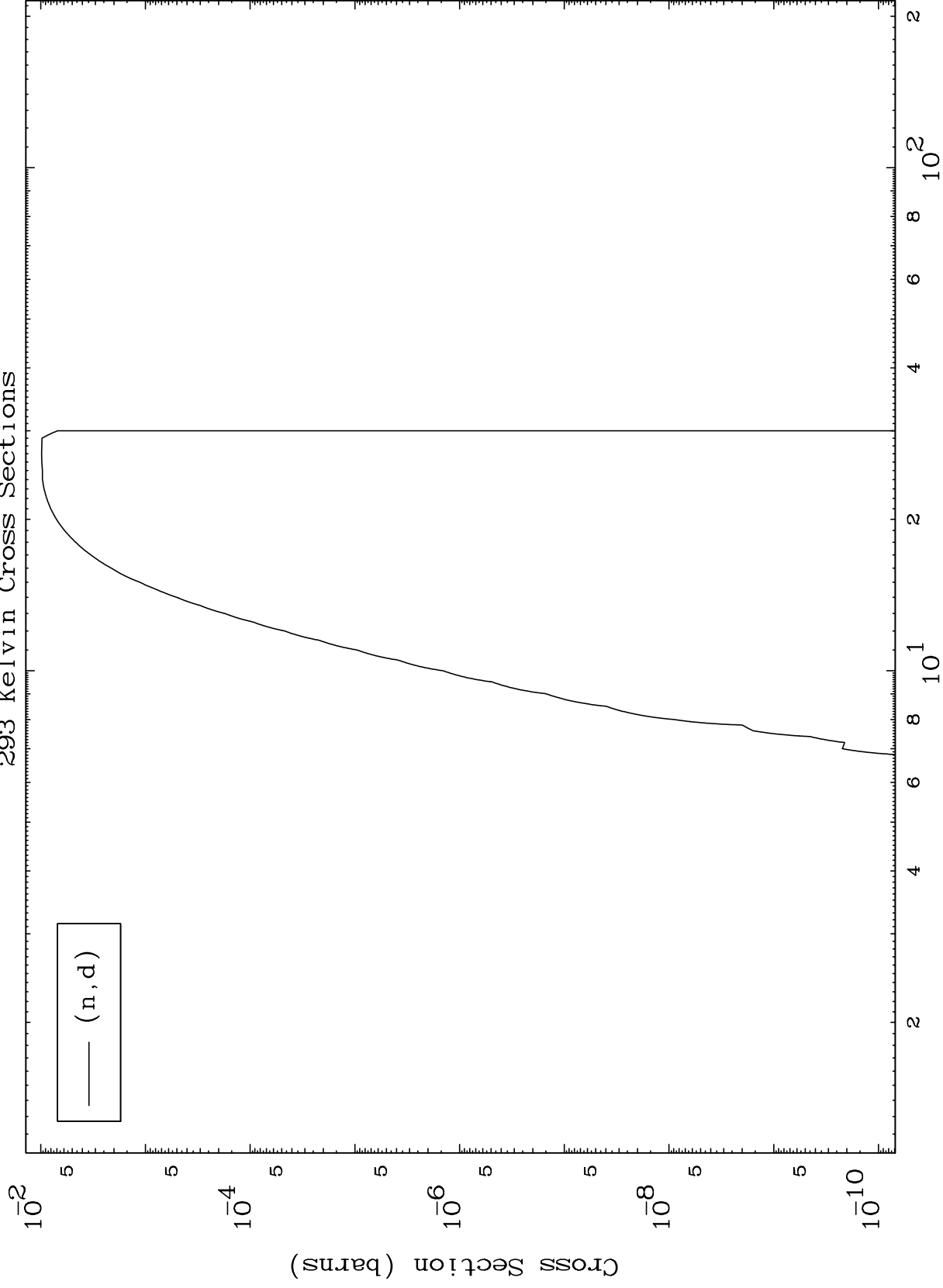
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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-243



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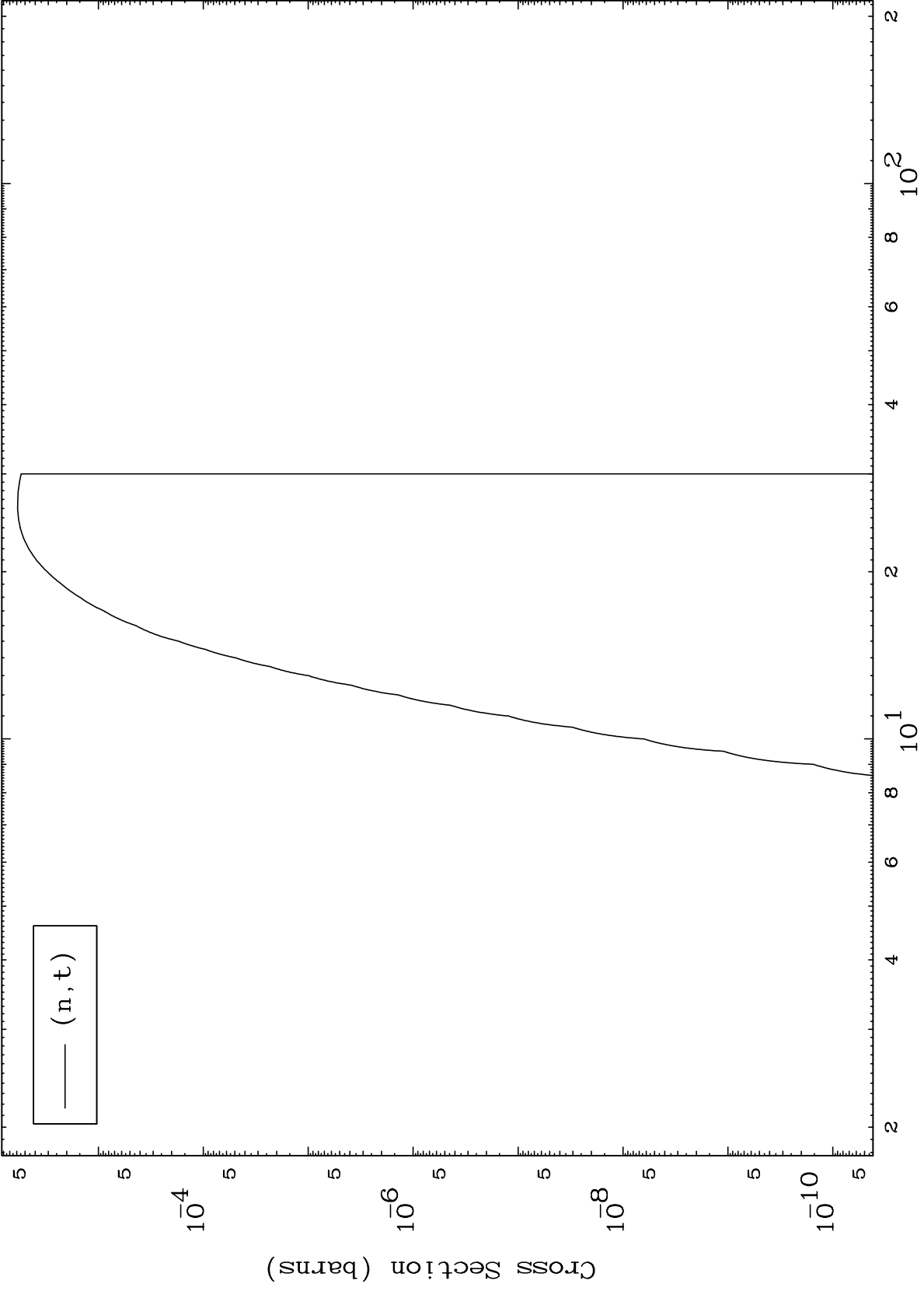
Incident Energy (MeV)

97-Bk-243

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(n,t) Levels  
293 Kelvin Cross Sections

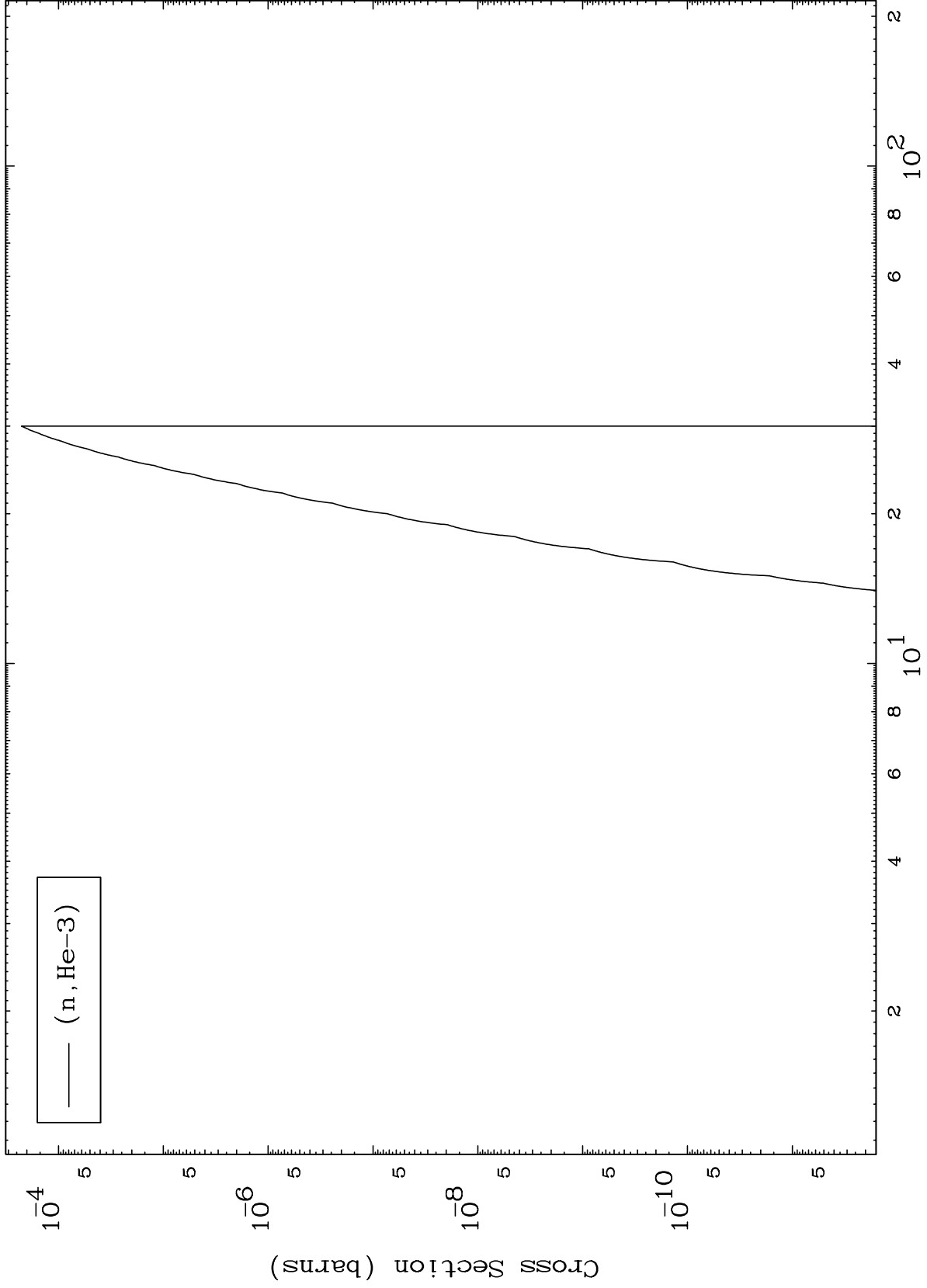
97-Bk-243



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(n,He3) Levels  
293 Kelvin Cross Sections

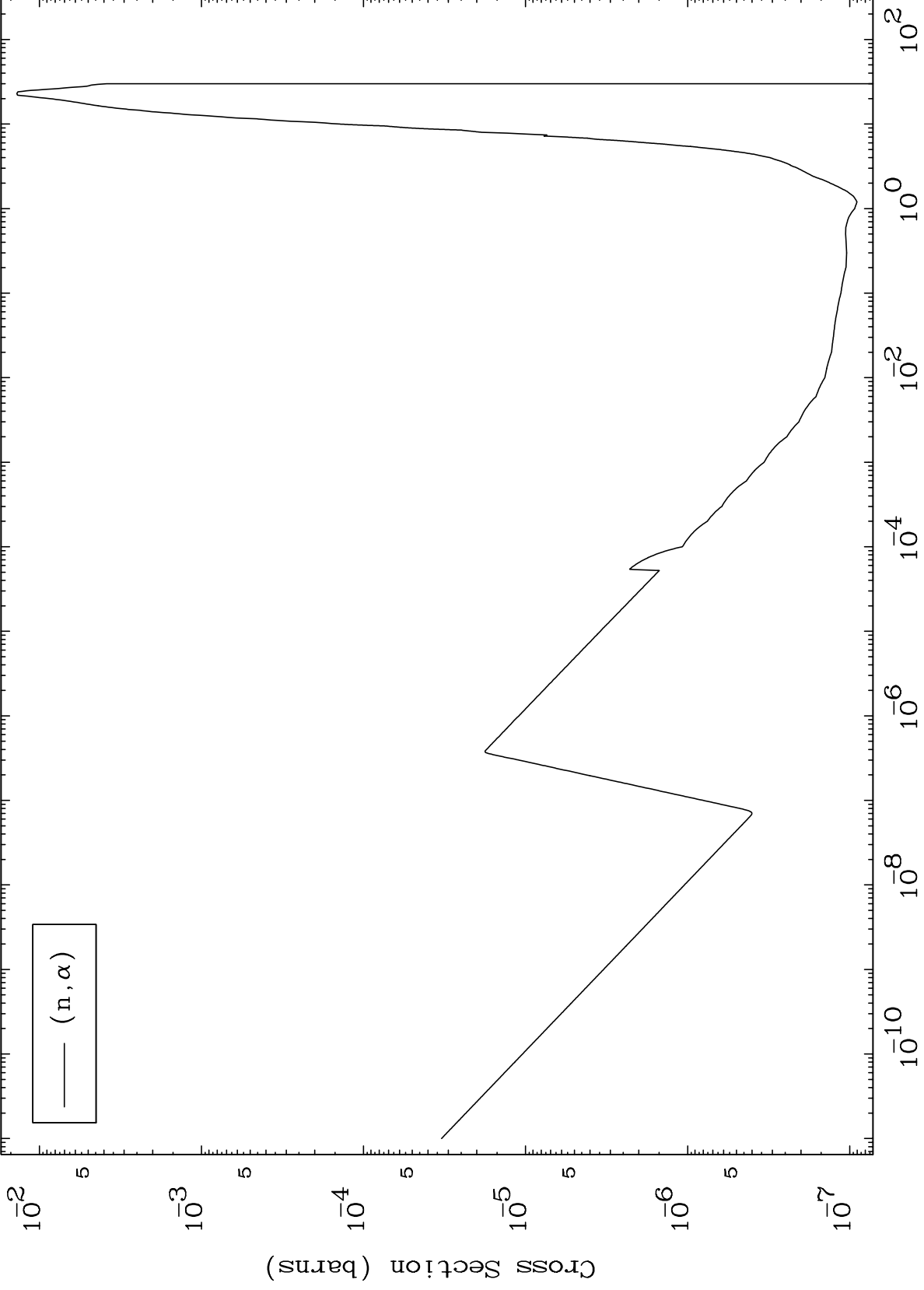
97-Bk-243



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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

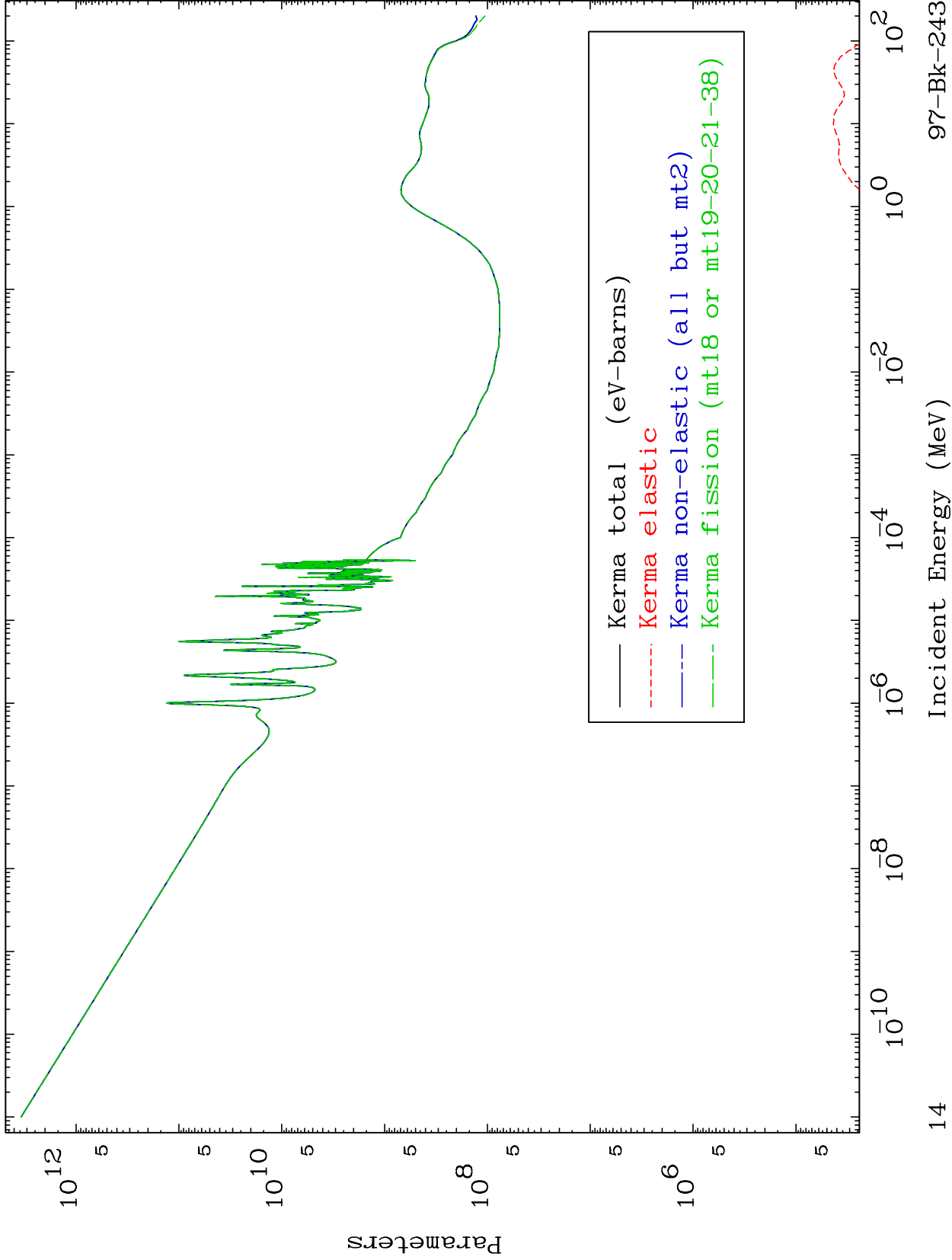
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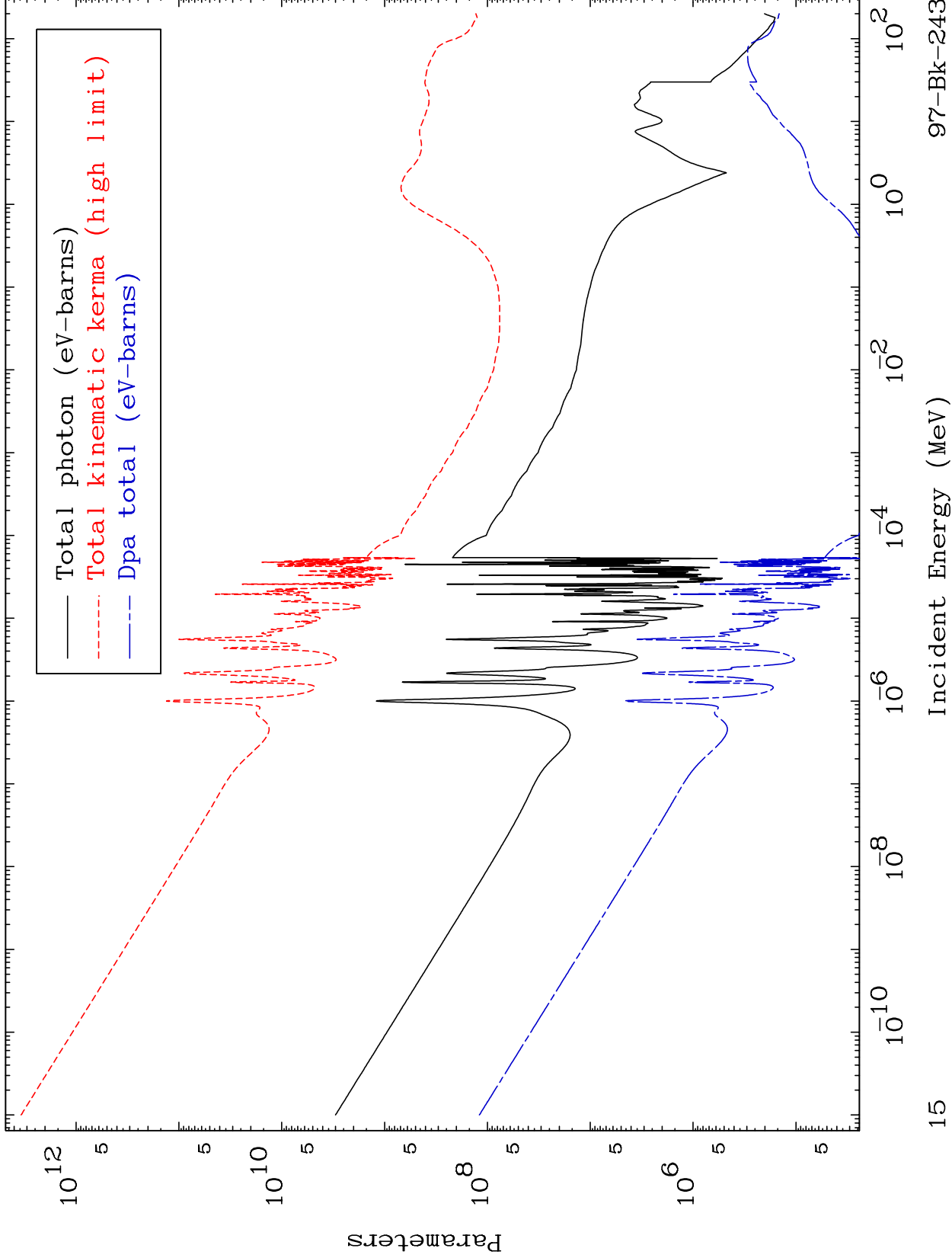


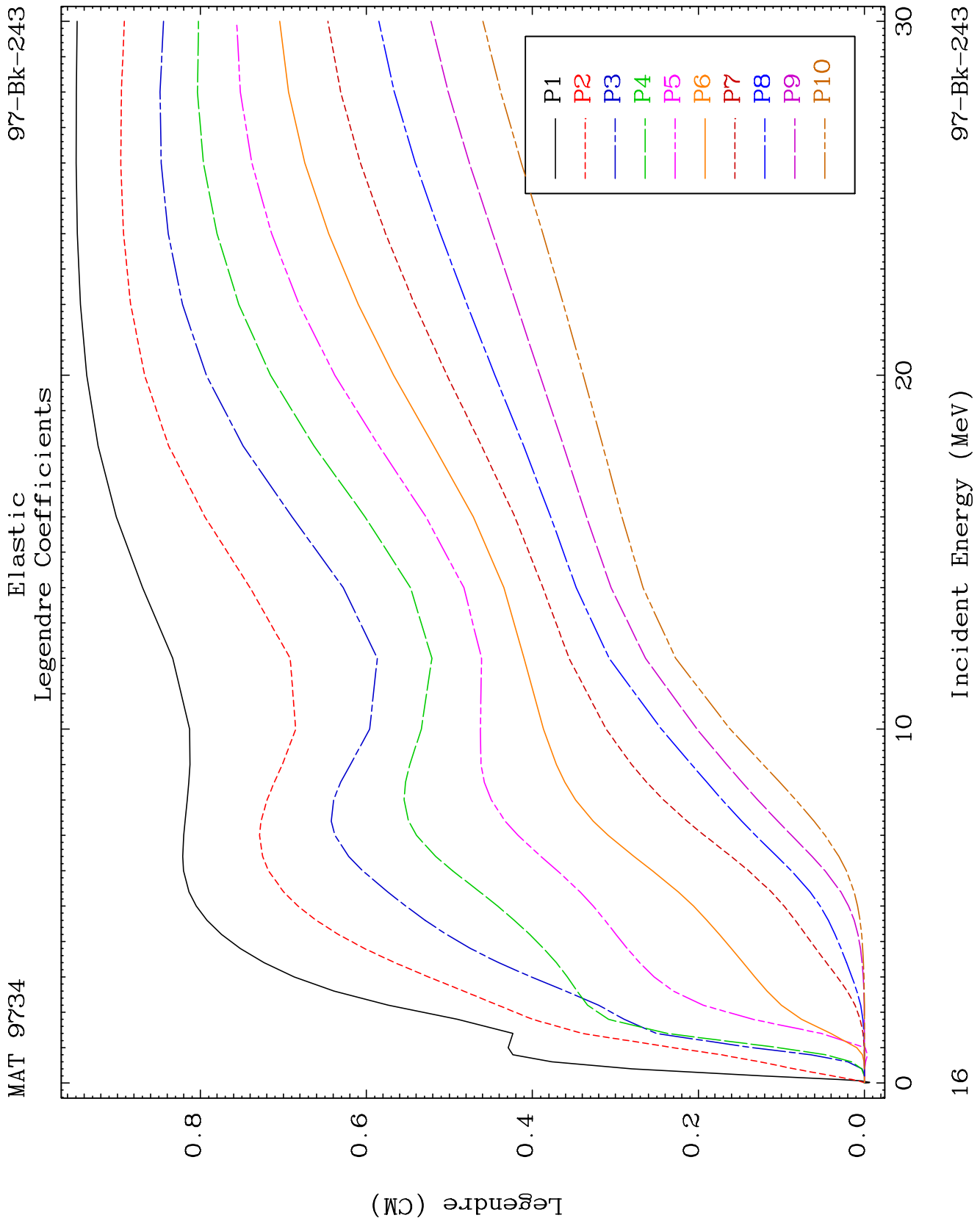
13

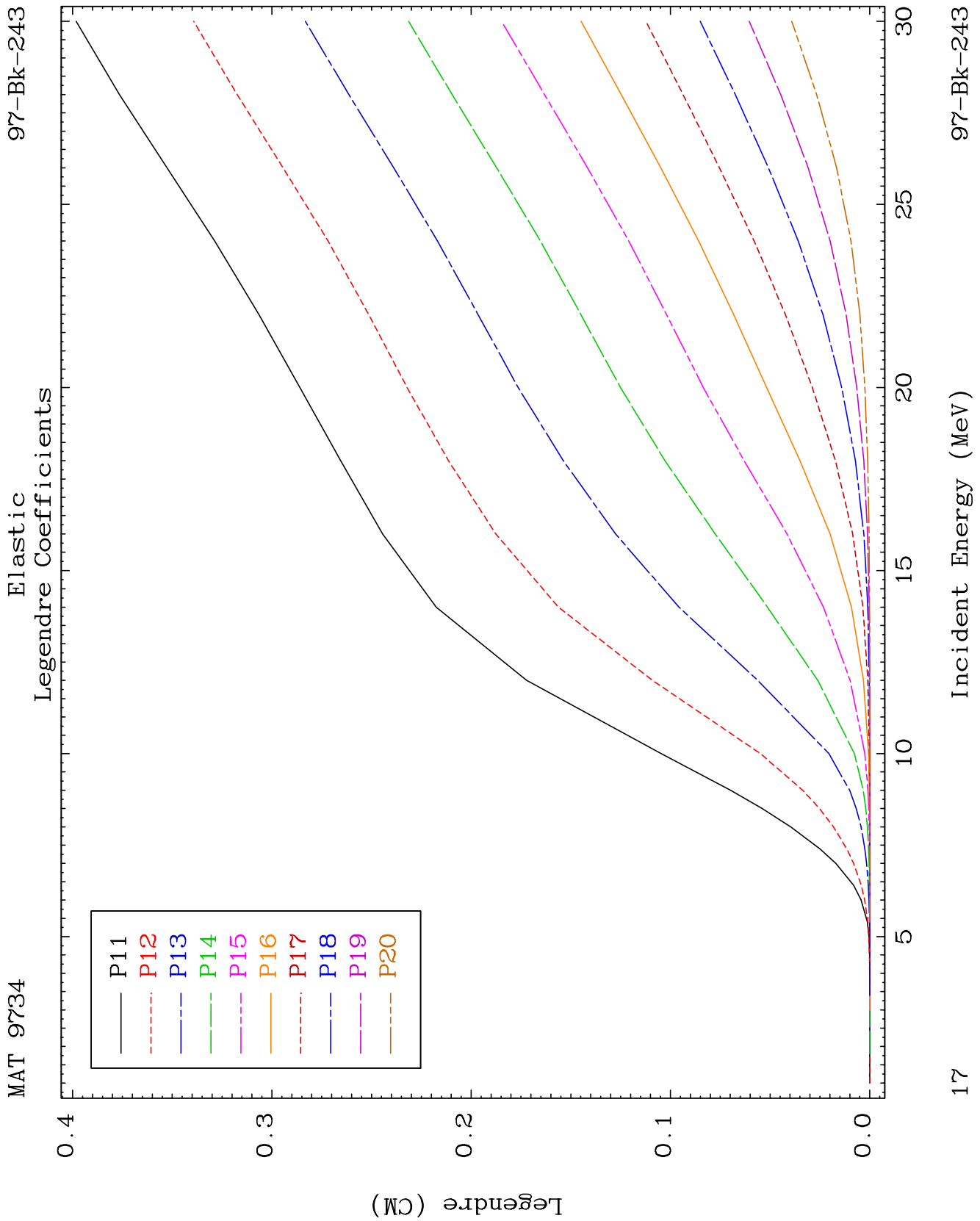
Incident Energy (MeV)

97-Bk-243





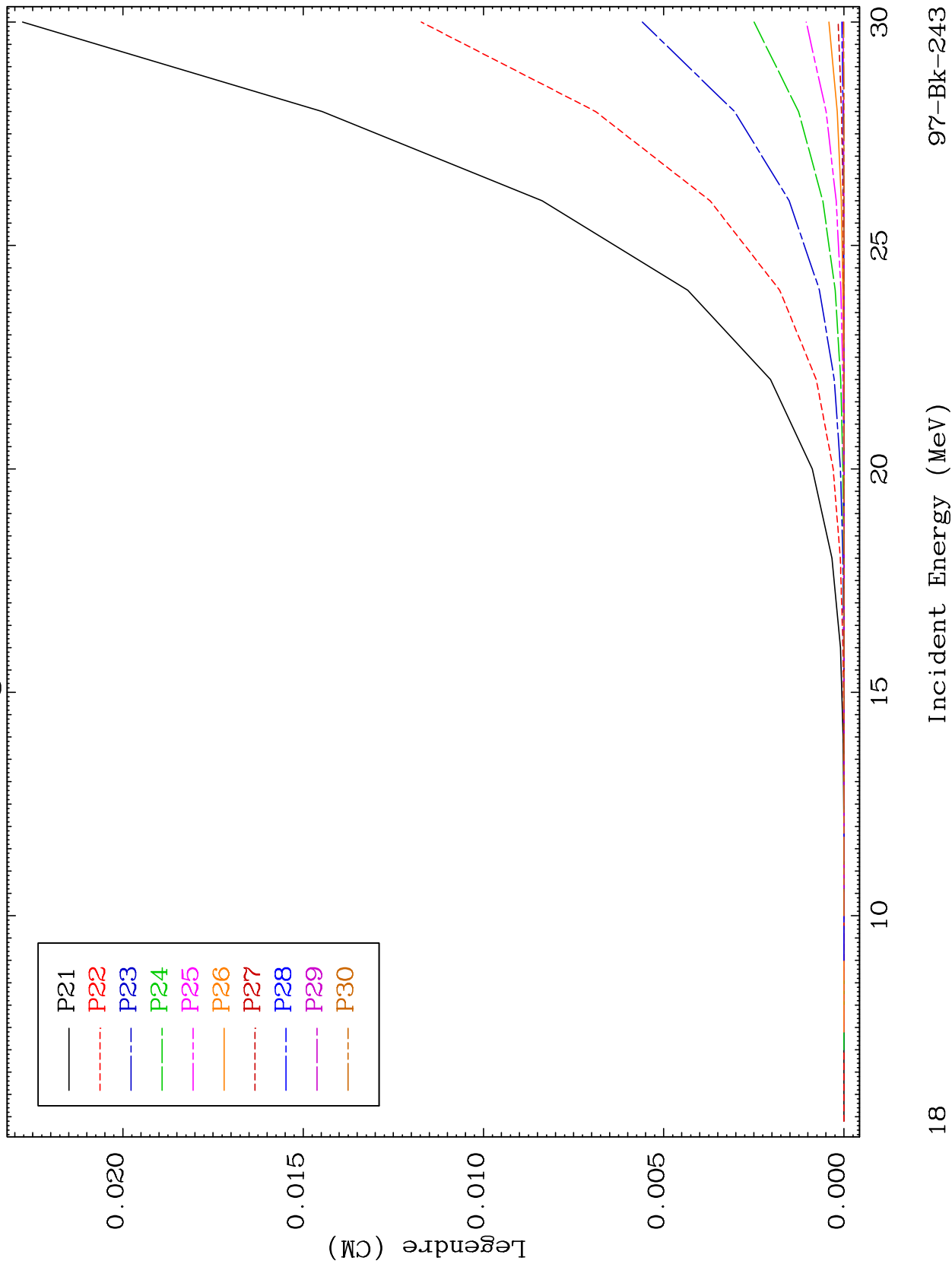




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### Elastic Legendre Coefficients

97-Bk-243



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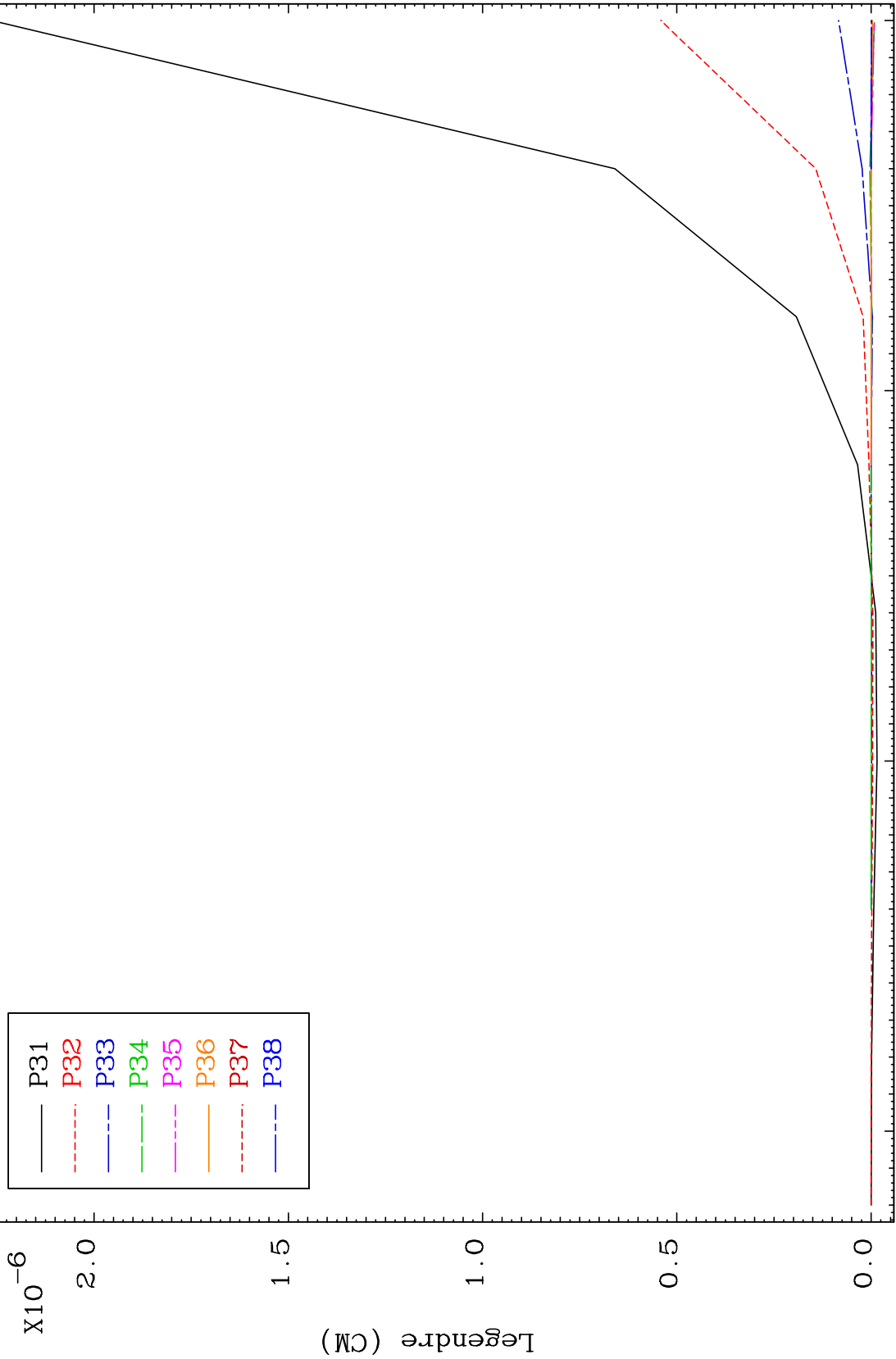
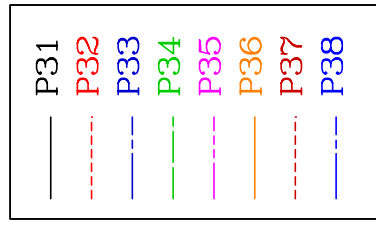
97-Bk-243

Incident Energy (MeV)

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Elastic Legendre Coefficients

97-Bk-243



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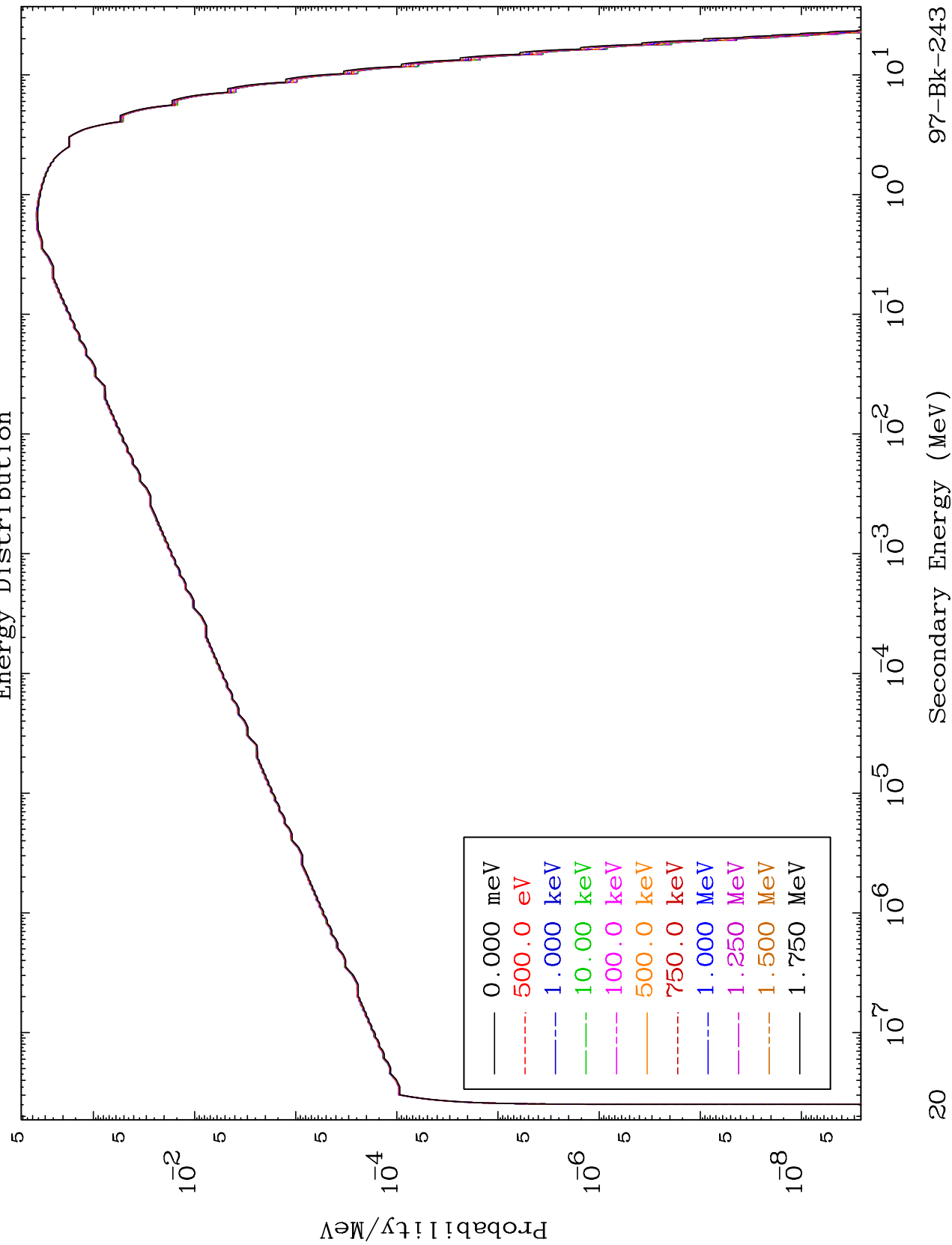
Incident Energy (MeV)

97-Bk-243

MAT 9734

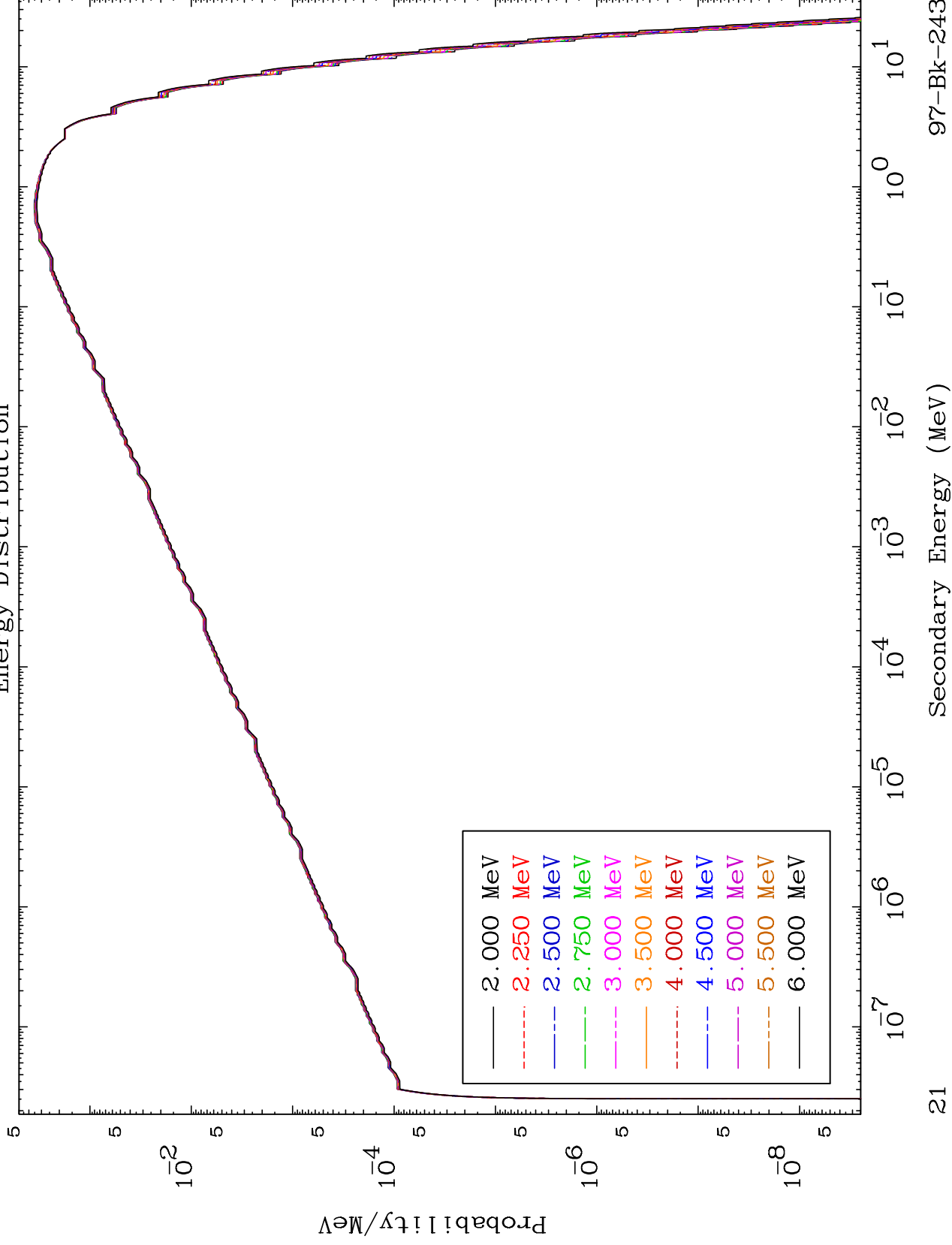
### Fission Energy Distribution

97-Bk-243

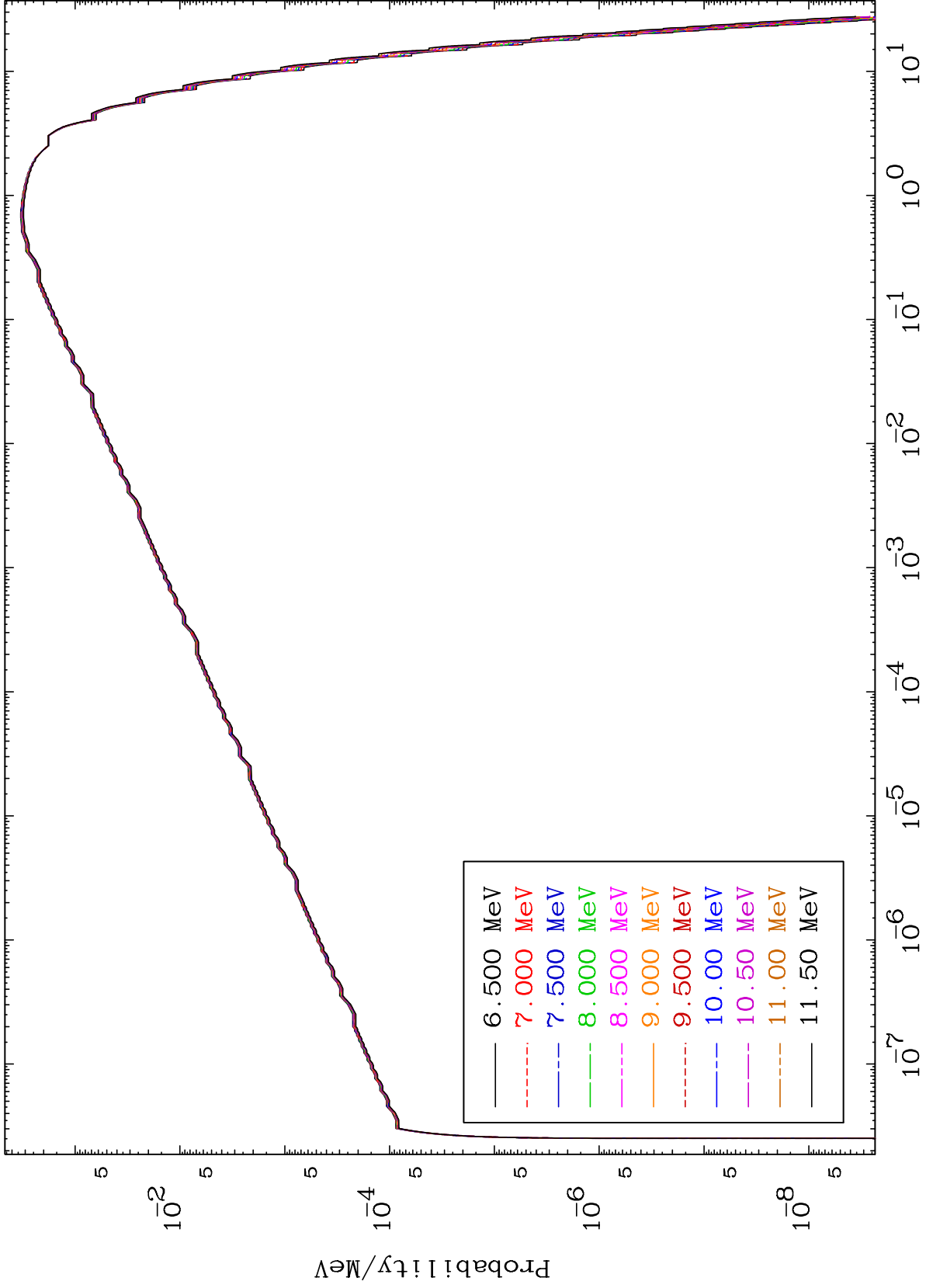


97-Bk-243

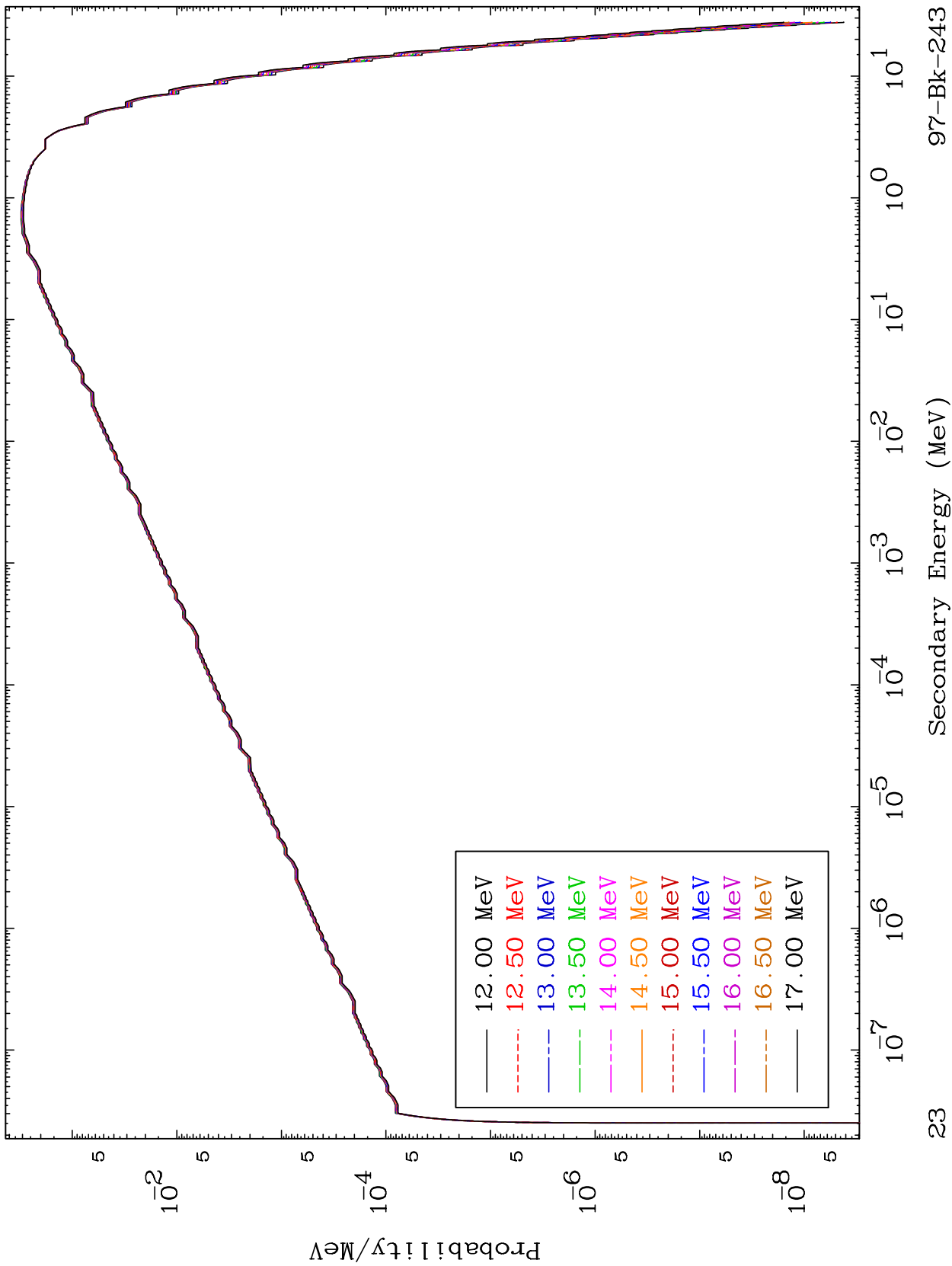
Fission  
Energy Distribution



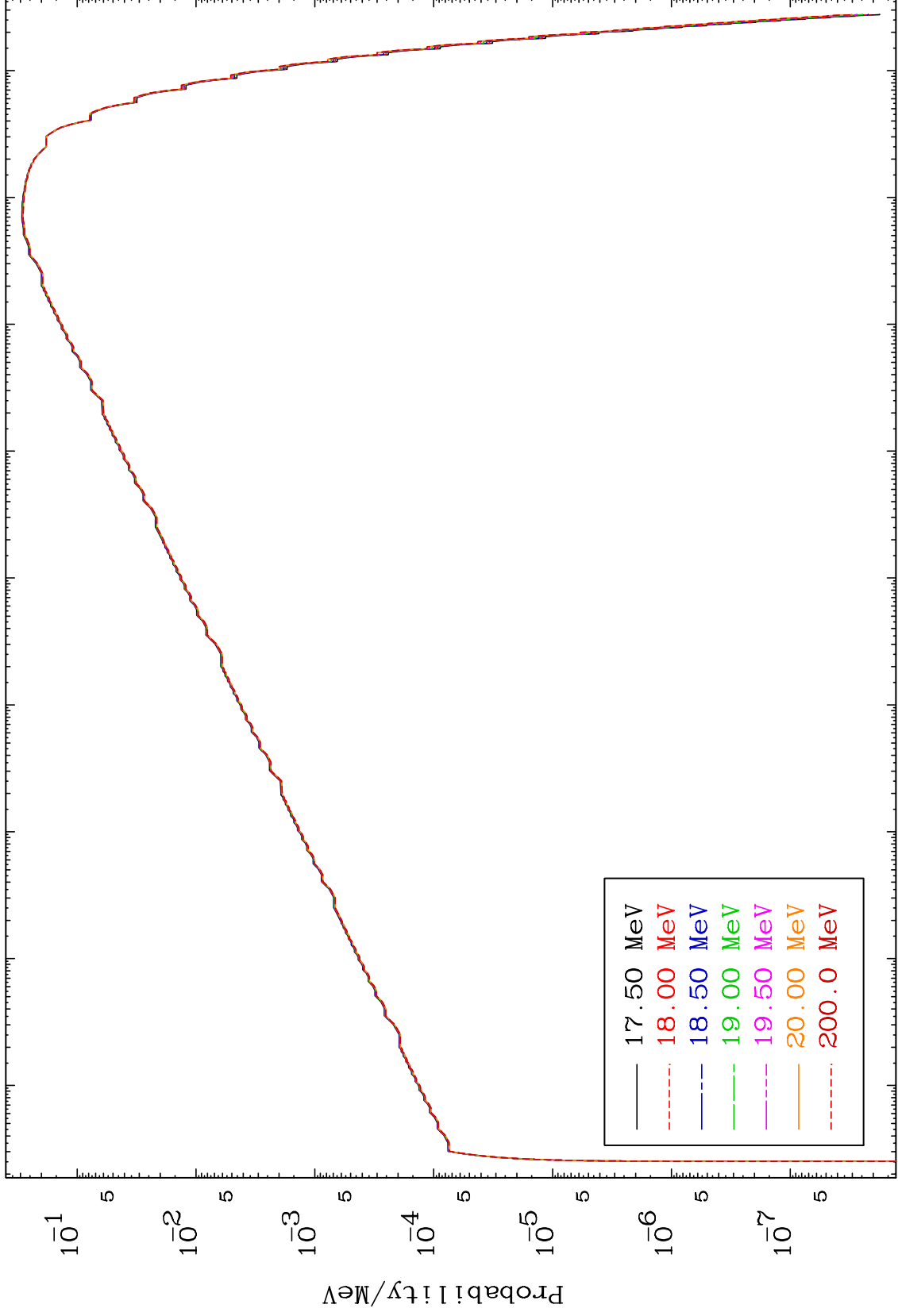
Fission  
Energy Distribution



Fission  
Energy Distribution



### Fission Energy Distribution

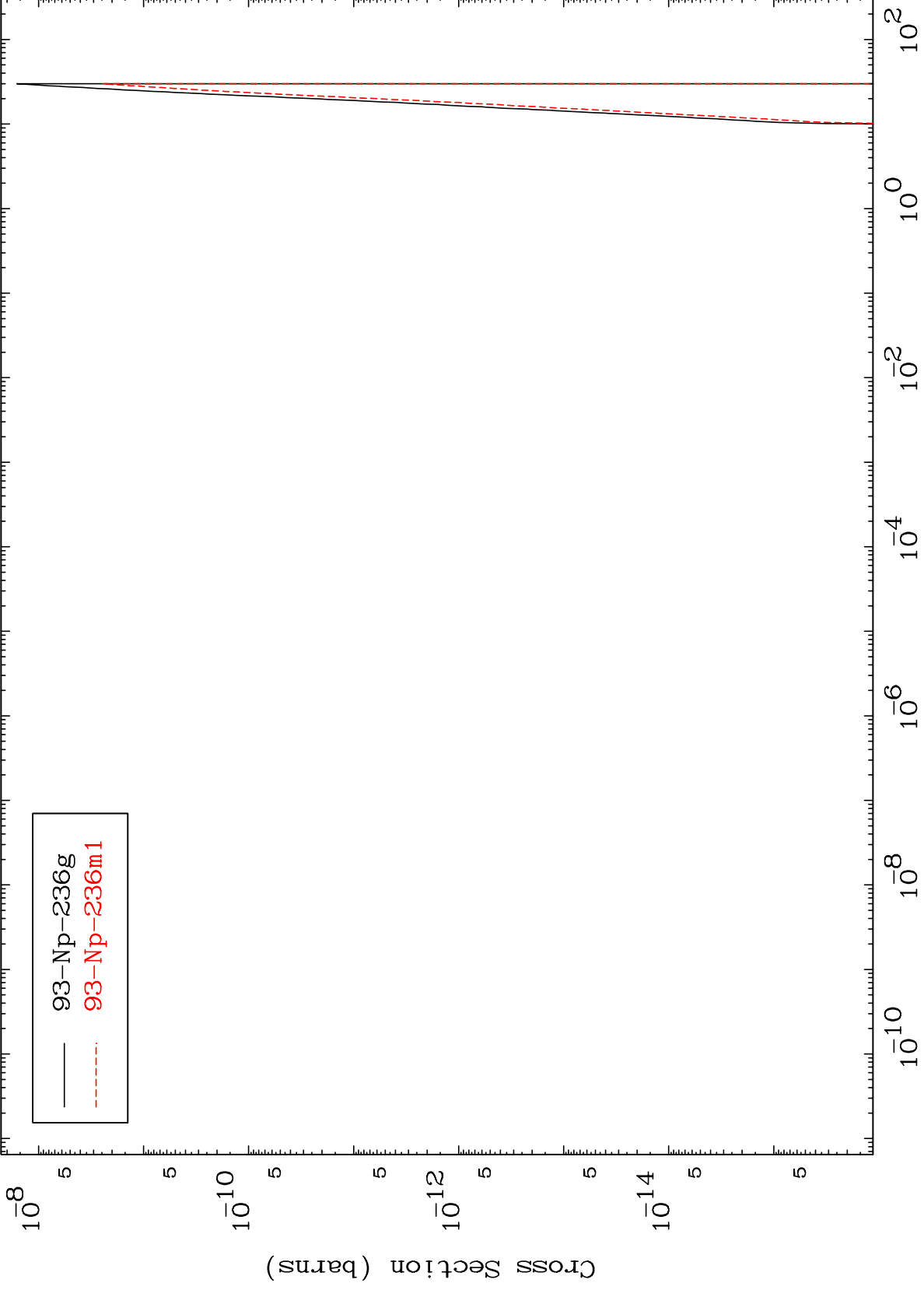


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(n,2α)

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Radionuclide Production Cross Section

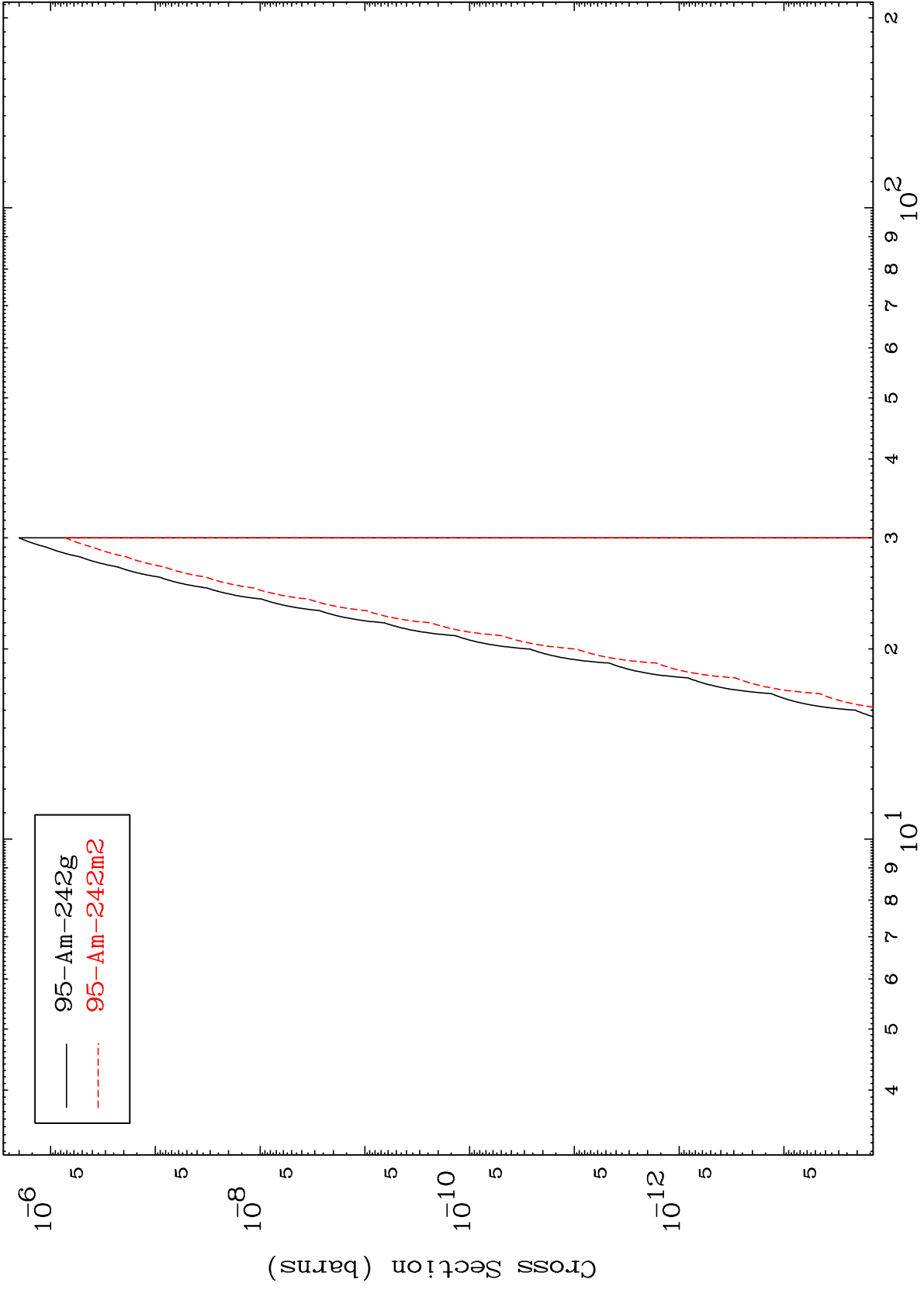


— 93-Np-236g  
- - - 93-Np-236m1

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97-Bk-243

(n,2p)  
Radionuclide Production Cross Section



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97-Bk-243

Incident Energy (MeV)