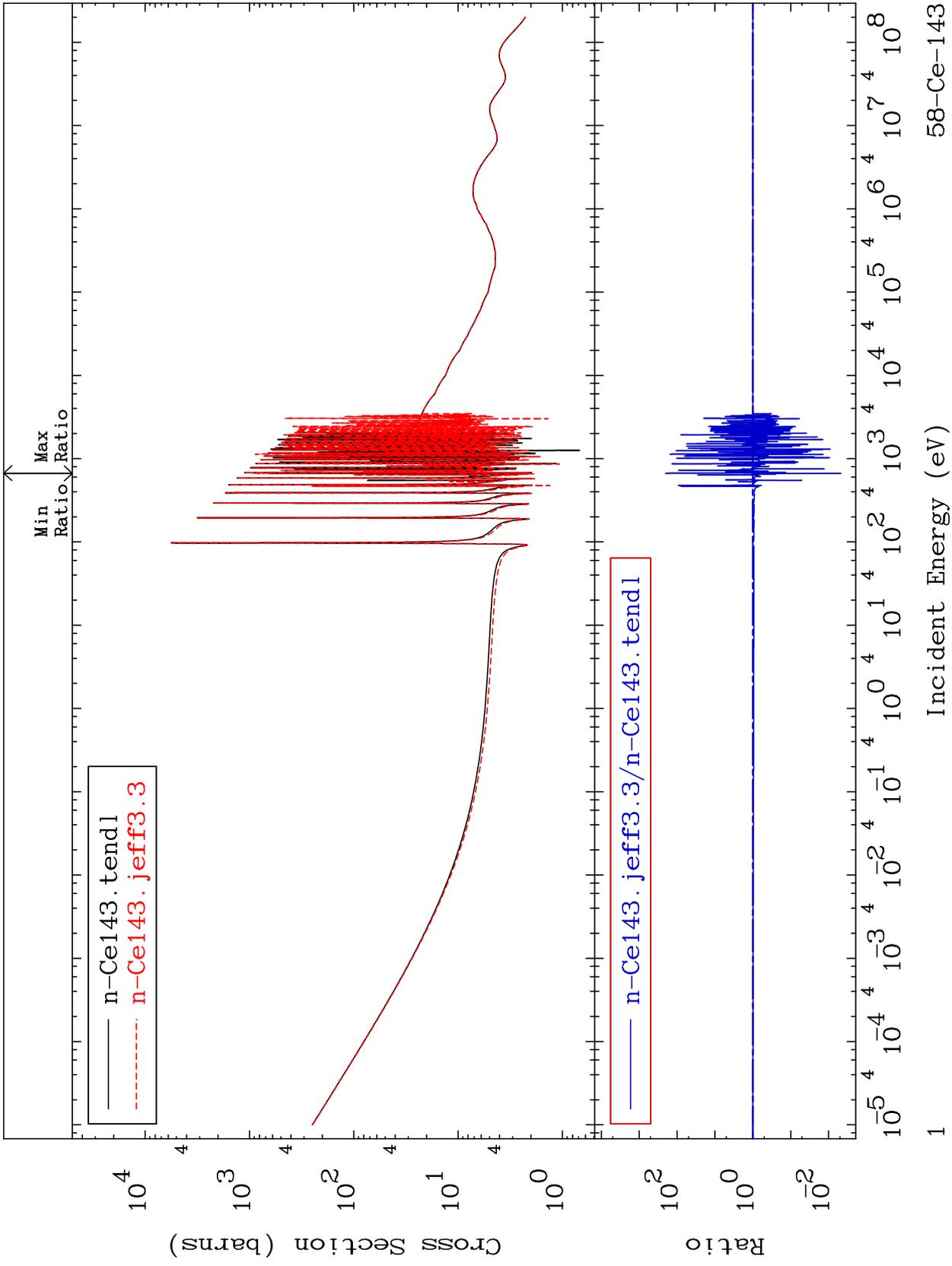


MAT 5846

Total Cross Section
58-Ce-143
-99.52 To 9999. %



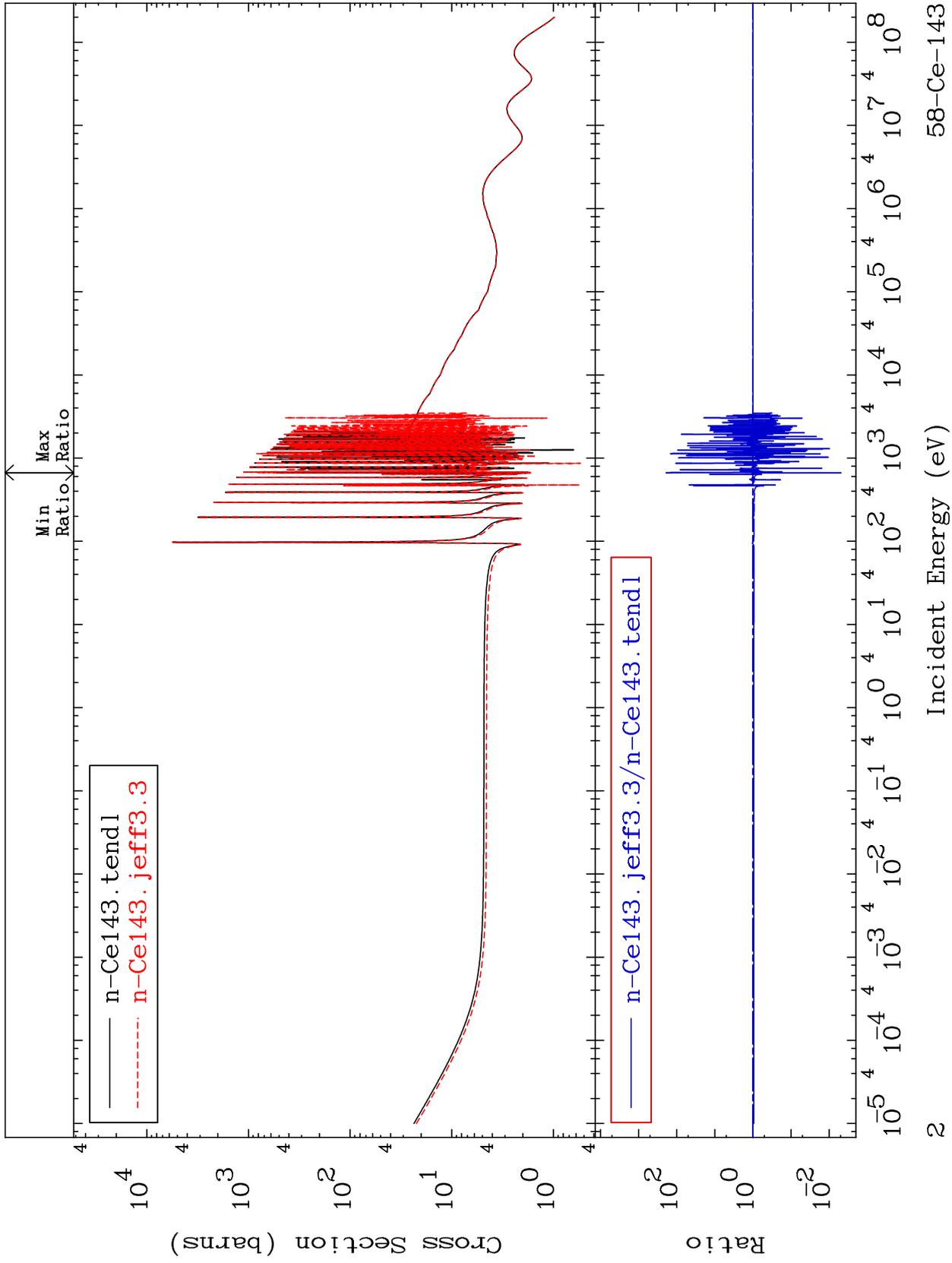
MAT 5846

Elastic

58-Ce-143

Cross Section

-99.51 To 9999. %



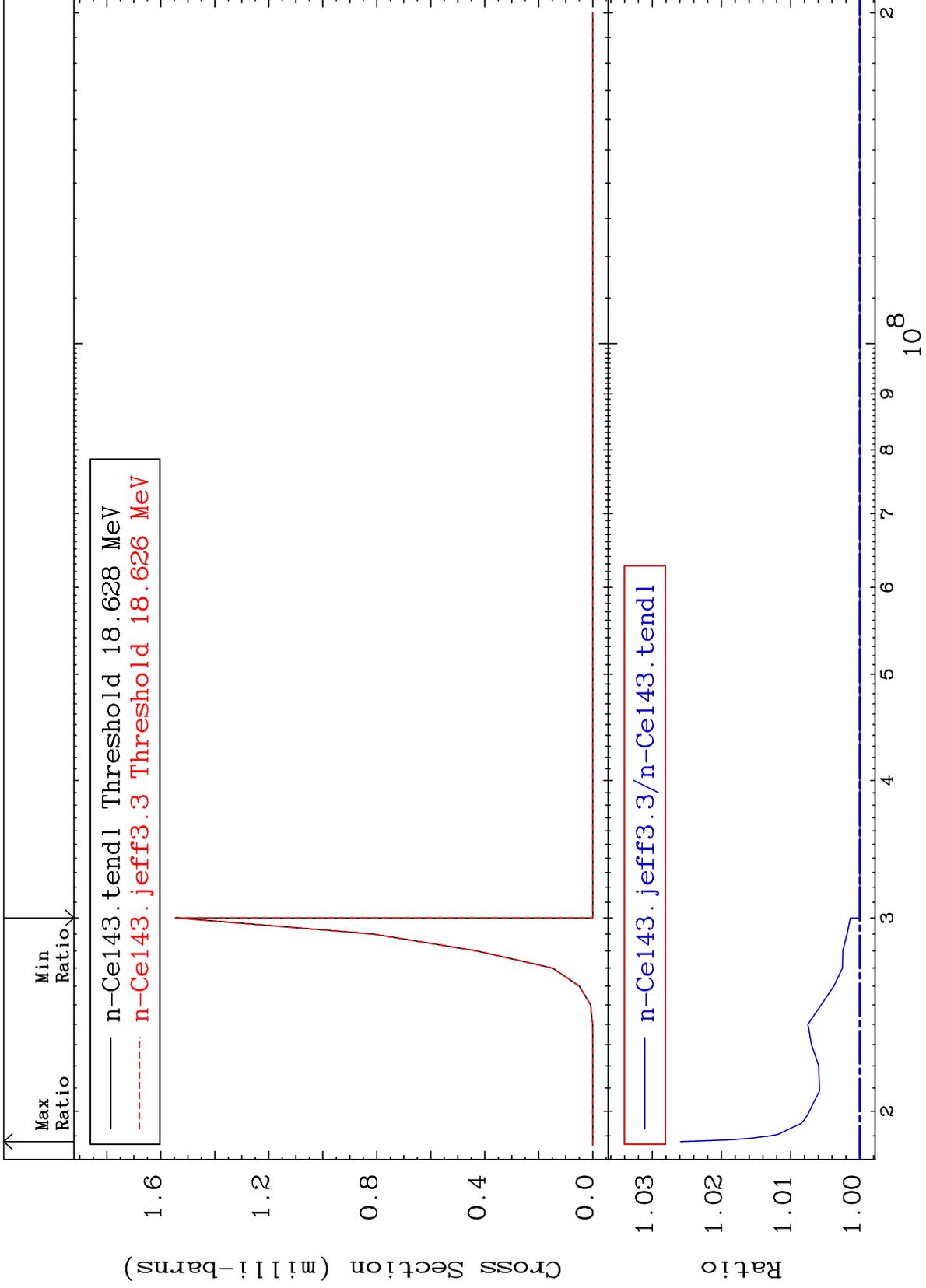
Incident Energy (eV)

58-Ce-143

MAT 5846

(n,2n) d
Cross Section

58-Ce-143
0.000 To 2.591 %



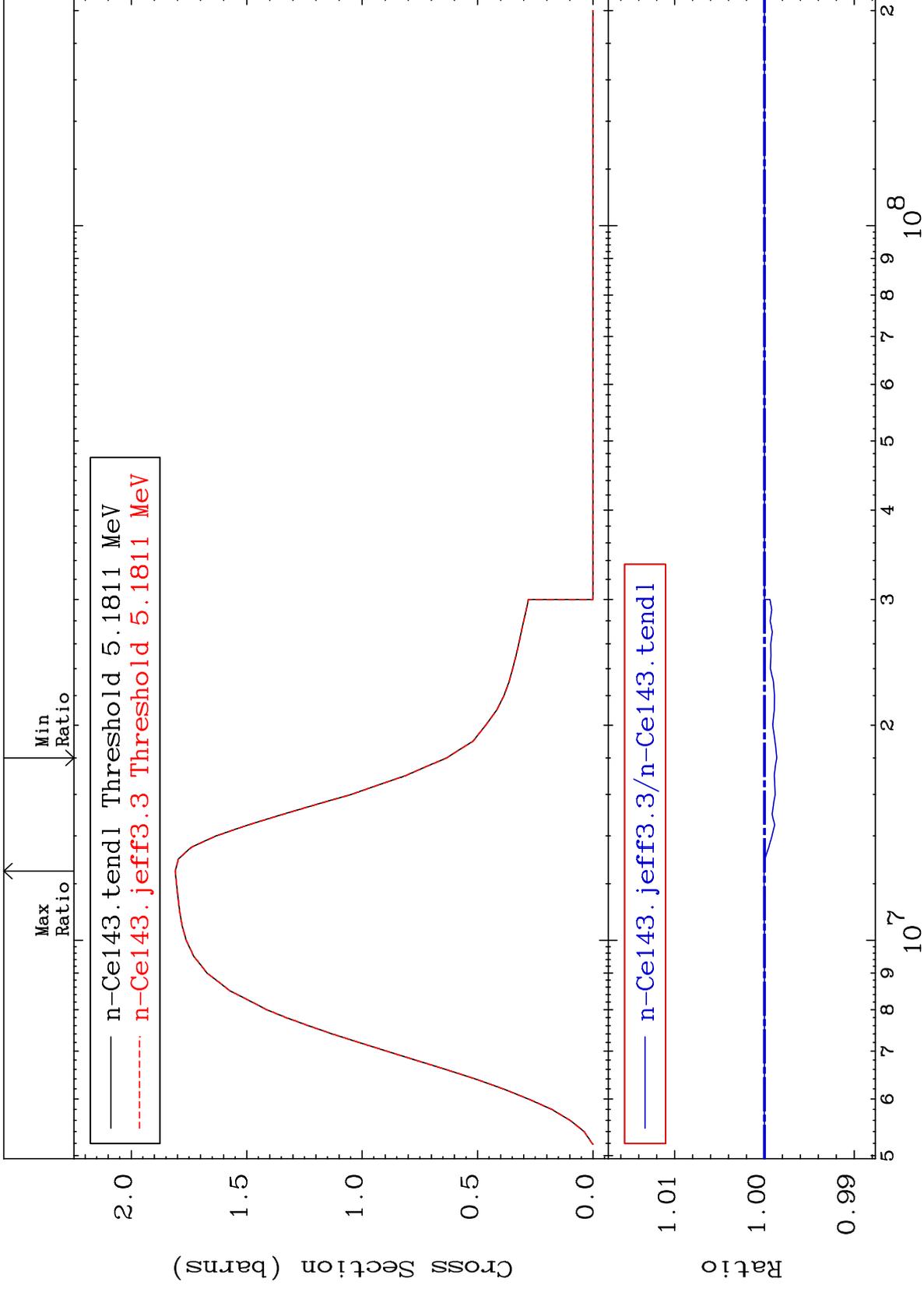
MAT 5846

(n,2n)

58-Ce-143

Cross Section

-0.137 To 0.004 %



— n-Ce143.tendl Threshold 5.1811 MeV
- - - n-Ce143.jeff3.3 Threshold 5.1811 MeV

— n-Ce143.jeff3.3/n-Ce143.tendl

MAT 5846

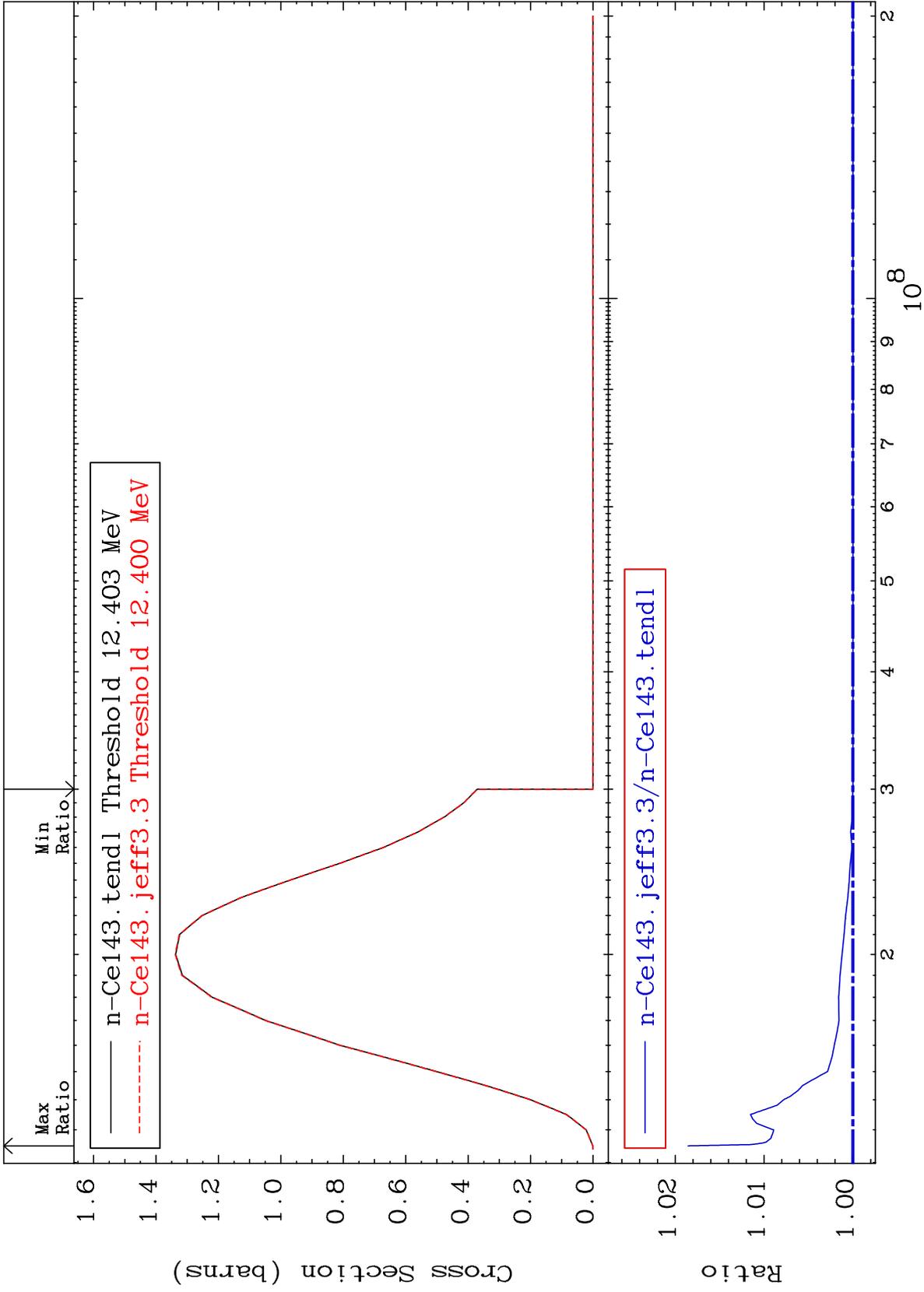
(n,3n)

58-Ce-143

Cross Section

0.000

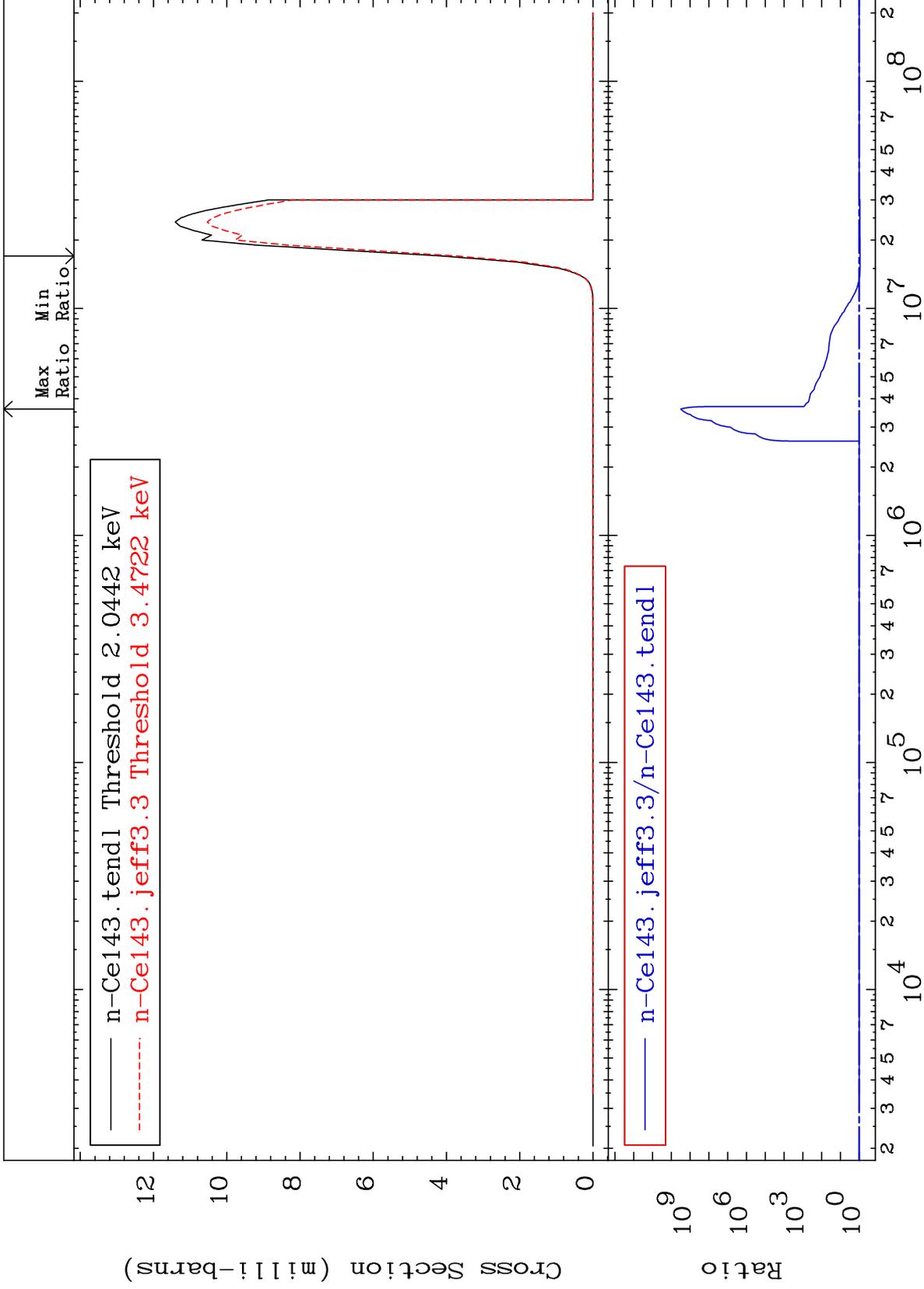
To 1.857 %



MAT 5846

$(n, n') \alpha$
Cross Section

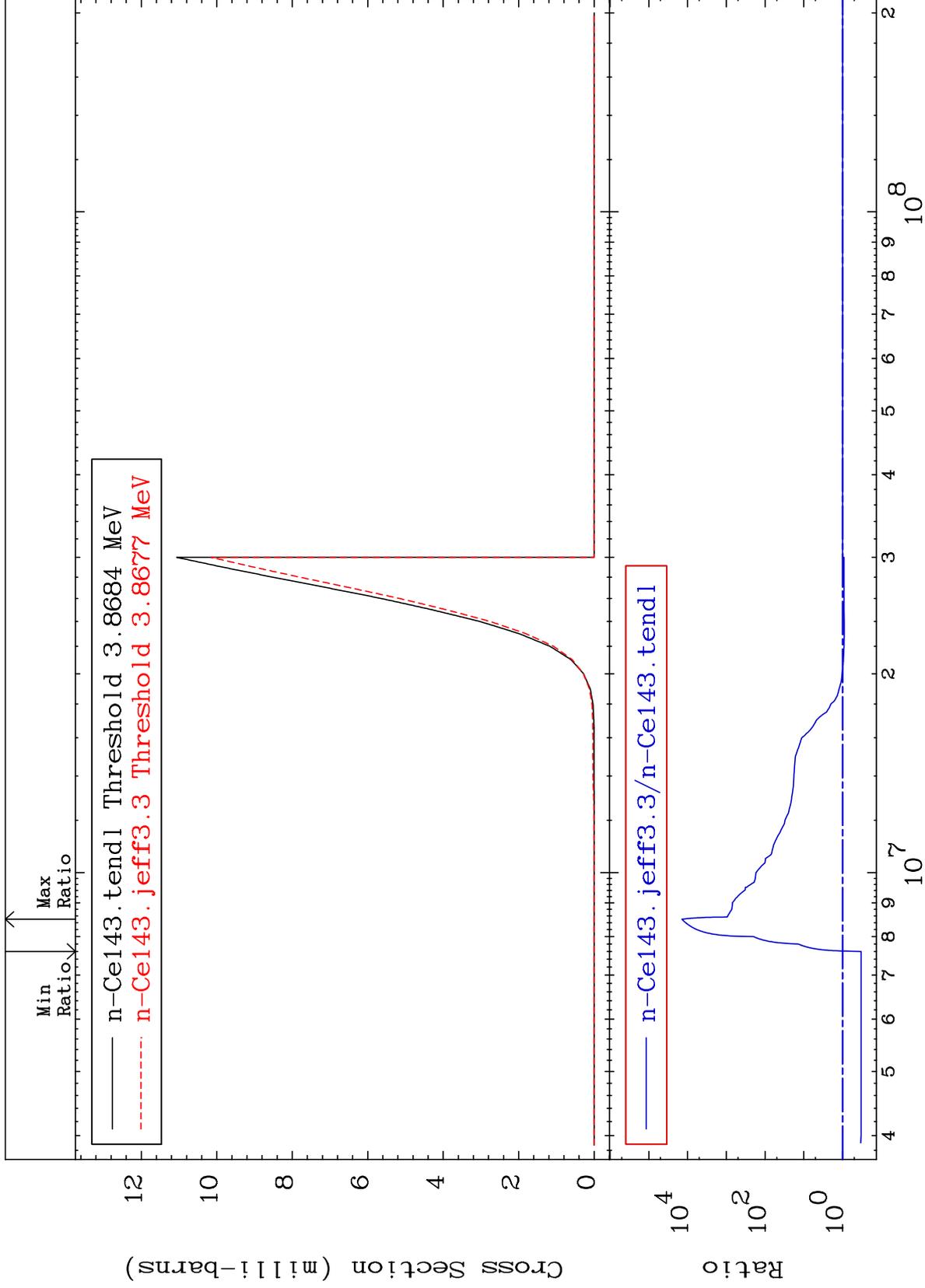
58-Ce-143
-9.761 To 9999. %



MAT 5846

(n,2n) α
Cross Section

58-Ce-143
-66.85 To 9999. %



7

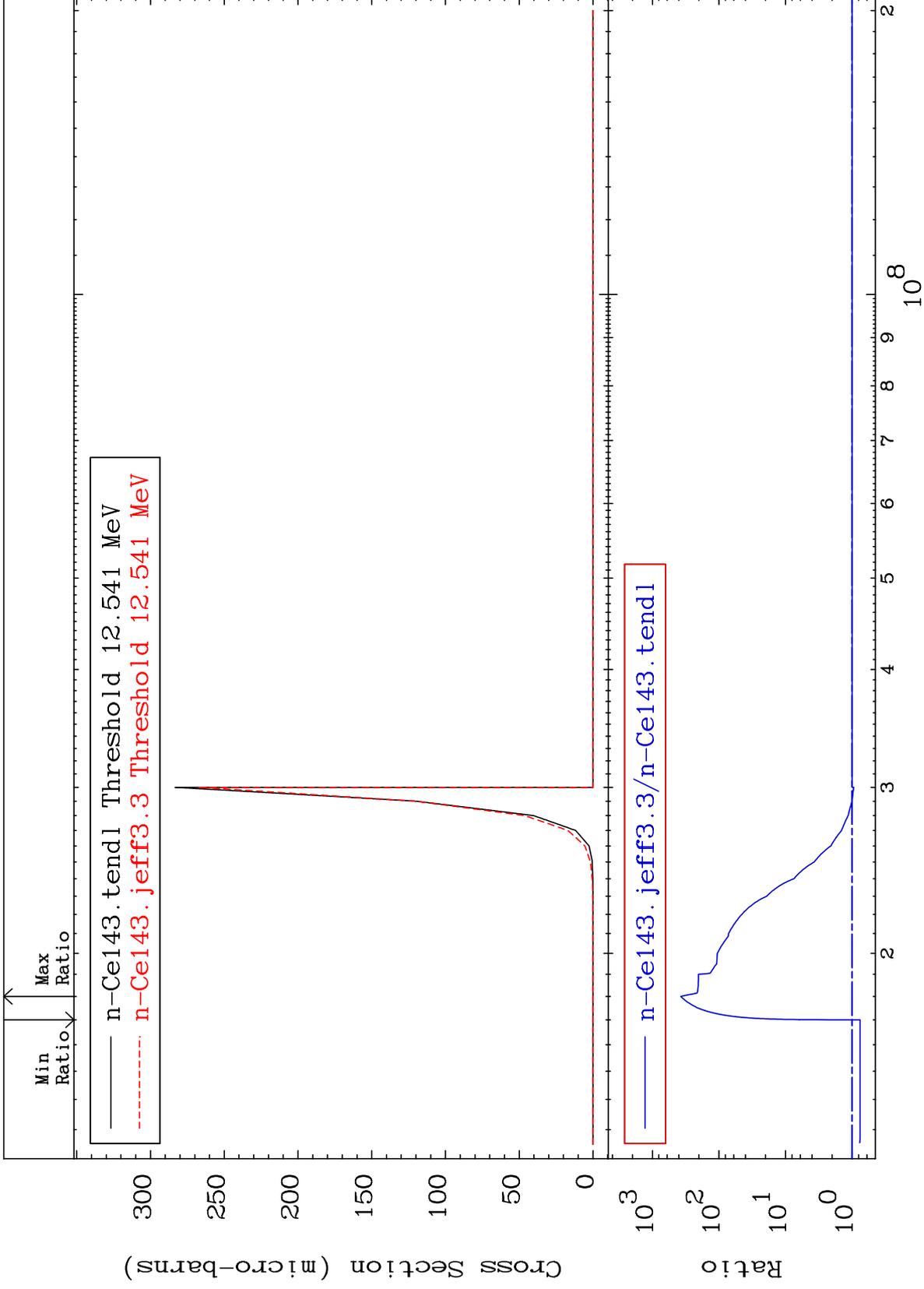
Incident Energy (eV)

58-Ce-143

MAT 5846

(n,3n) α
Cross Section

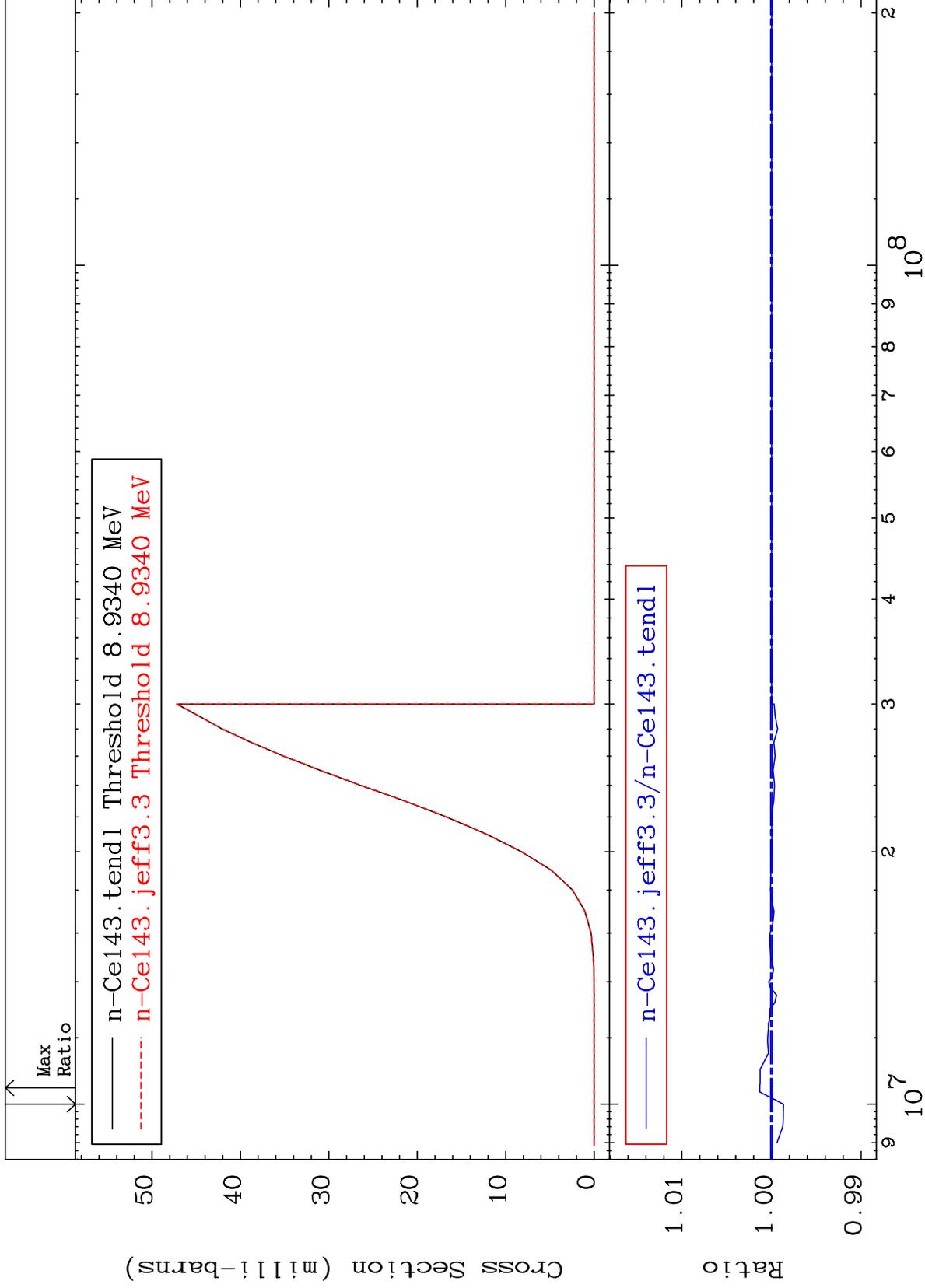
58-Ce-143
-24.11 To 9999. %



MAT 5846

(n,n') p
Cross Section

58-Ce-143
-0.133 To 0.132 %



58-Ce-143

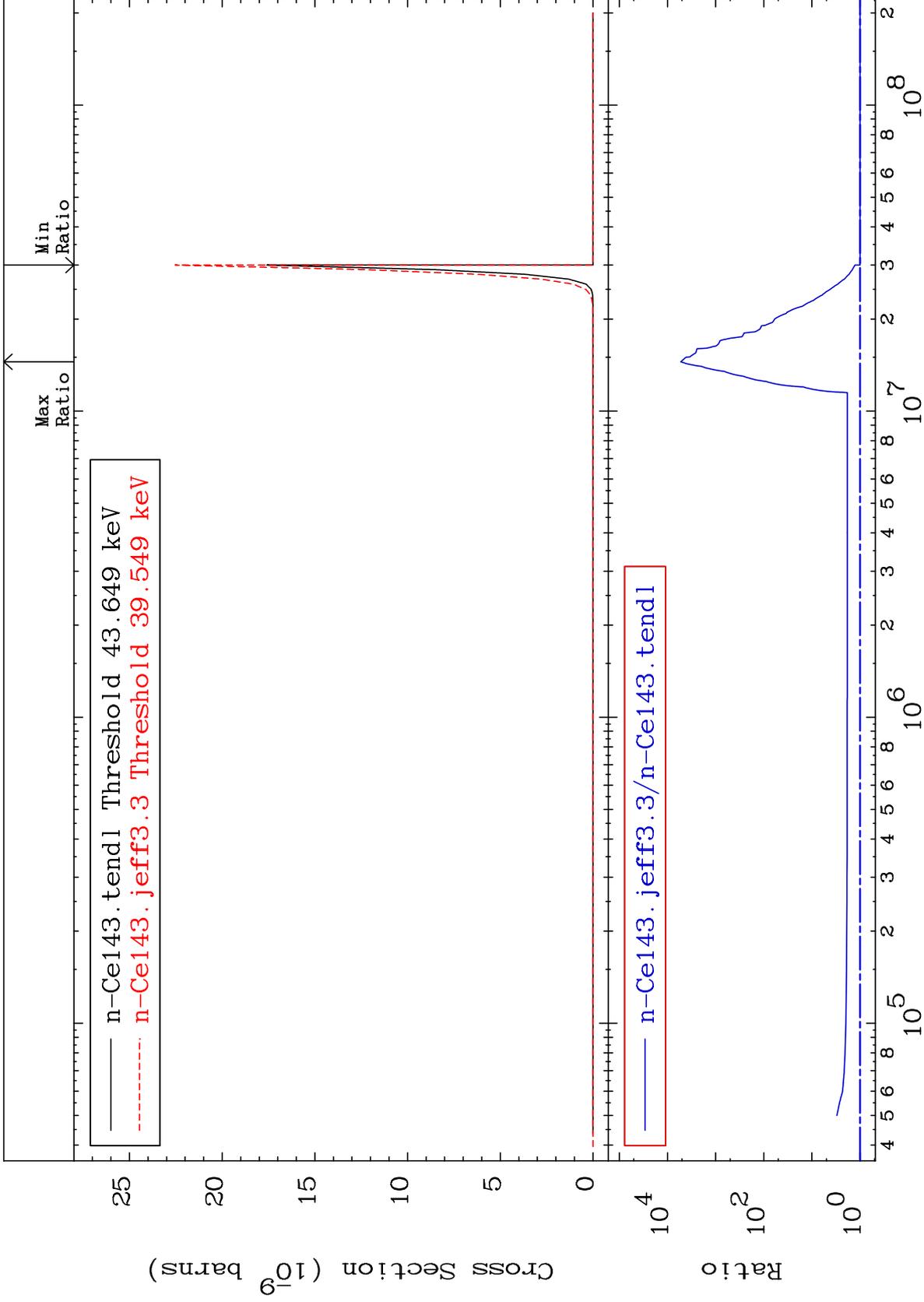
Incident Energy (eV)

9

MAT 5846

(n, n') 2α
Cross Section

58-Ce-143
To 9999. %



MAT 5846

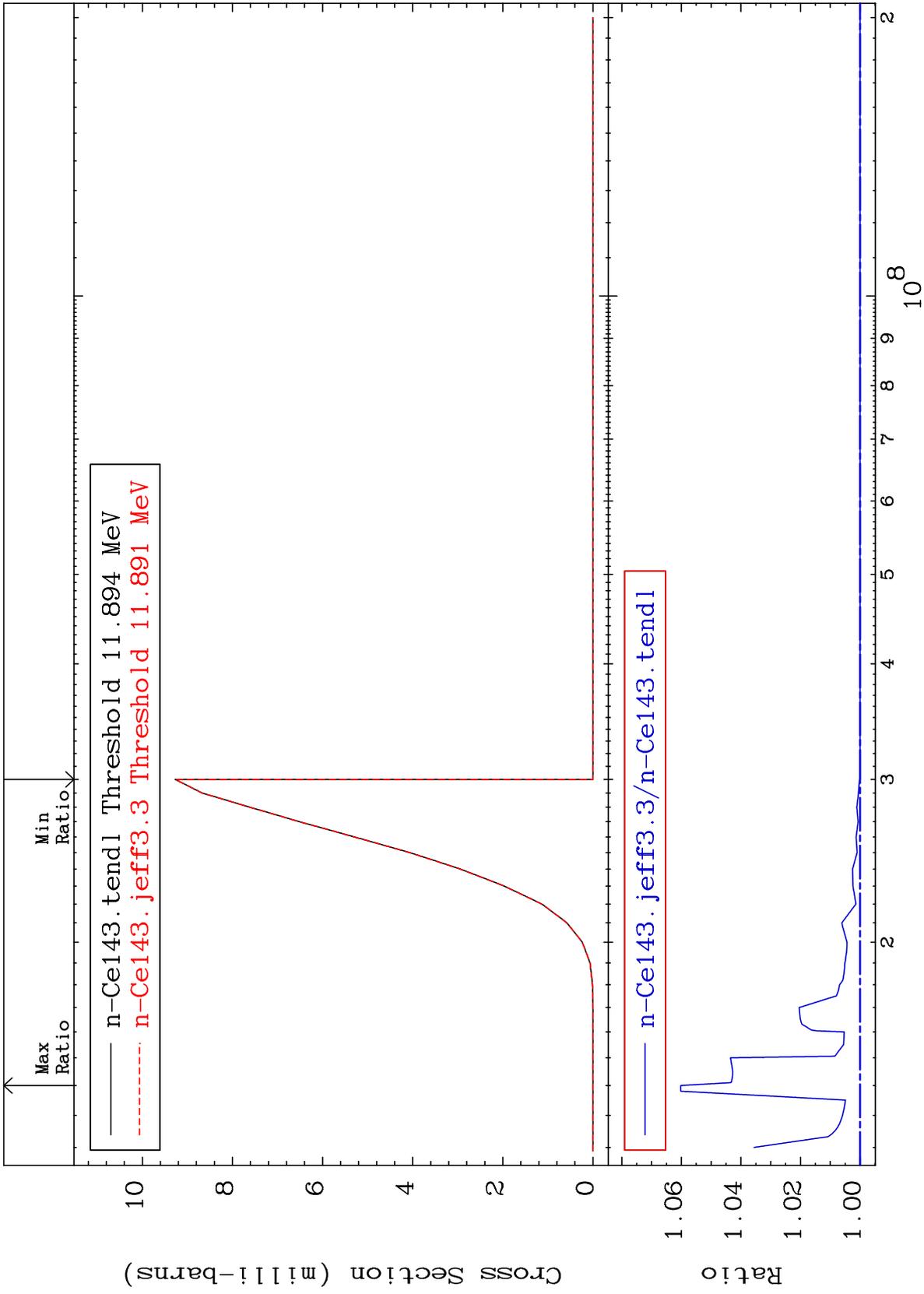
(n,n') d

58-Ce-143

Cross Section

0.000

To 6.020 %



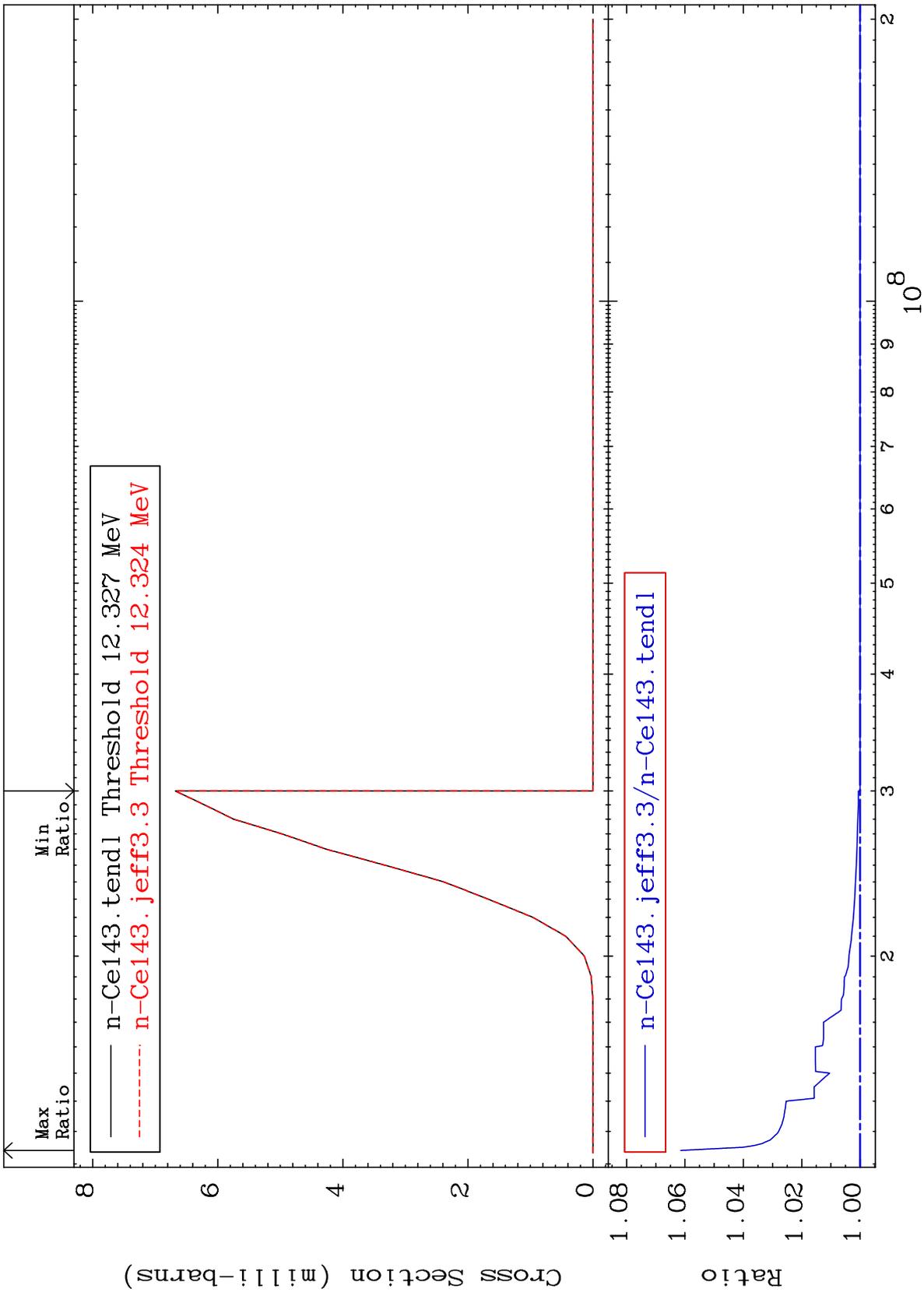
MAT 5846

(n,n') t

58-Ce-143

Cross Section

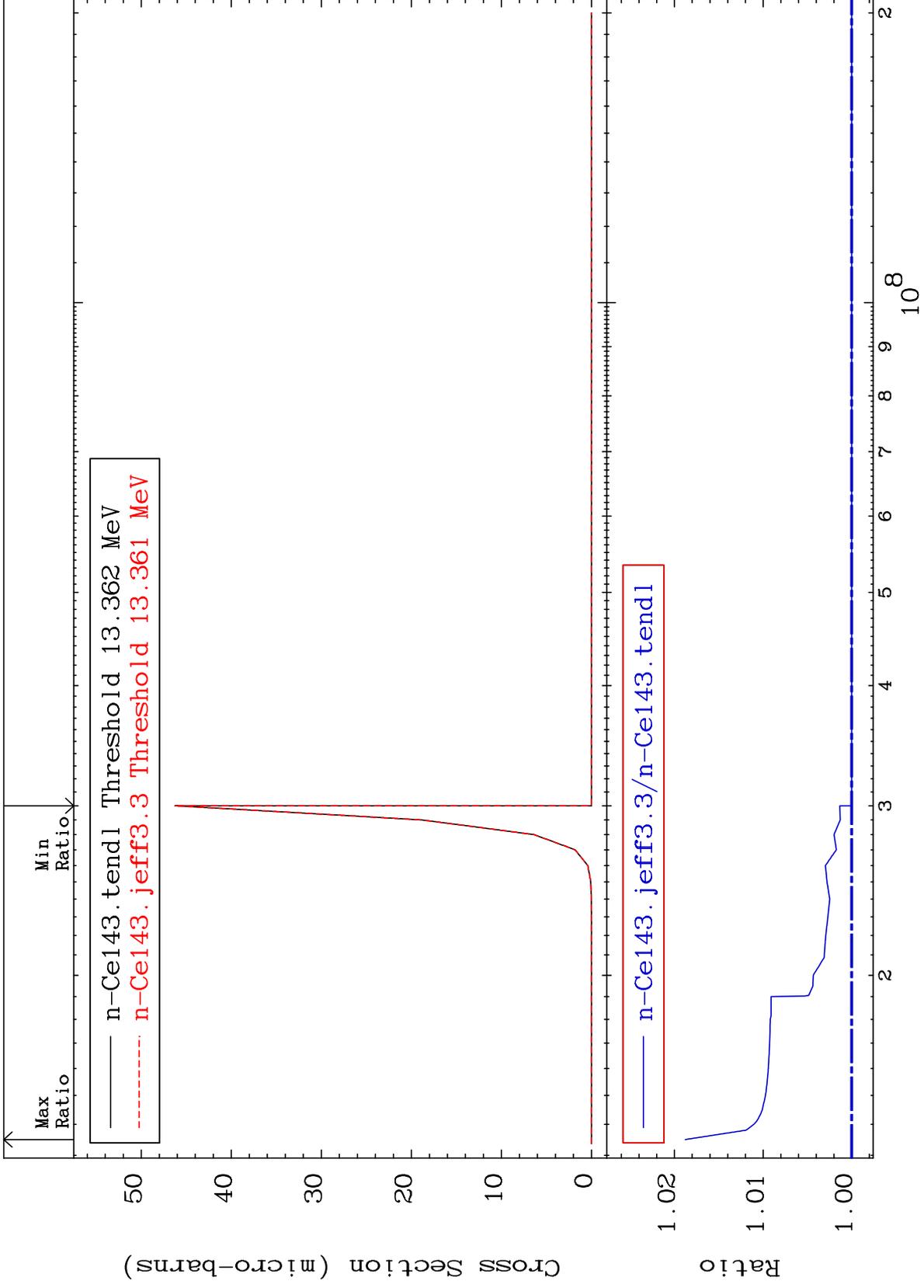
0.000 To 6.145 %



MAT 5846

(n, n') He-3
Cross Section

58-Ce-143
To 1.878 %



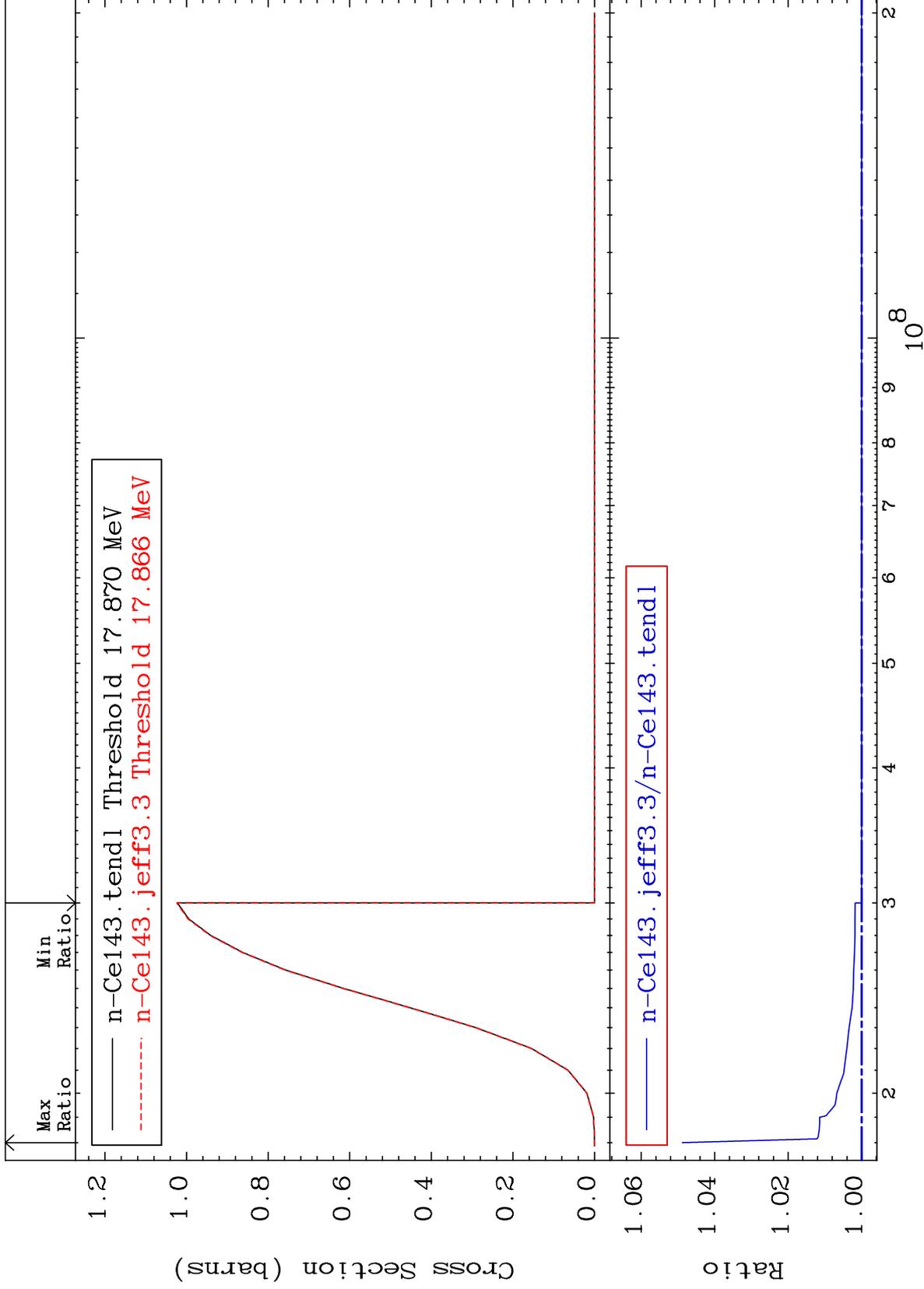
MAT 5846

(n,4n)

58-Ce-143

Cross Section

0.000 To 4.881 %



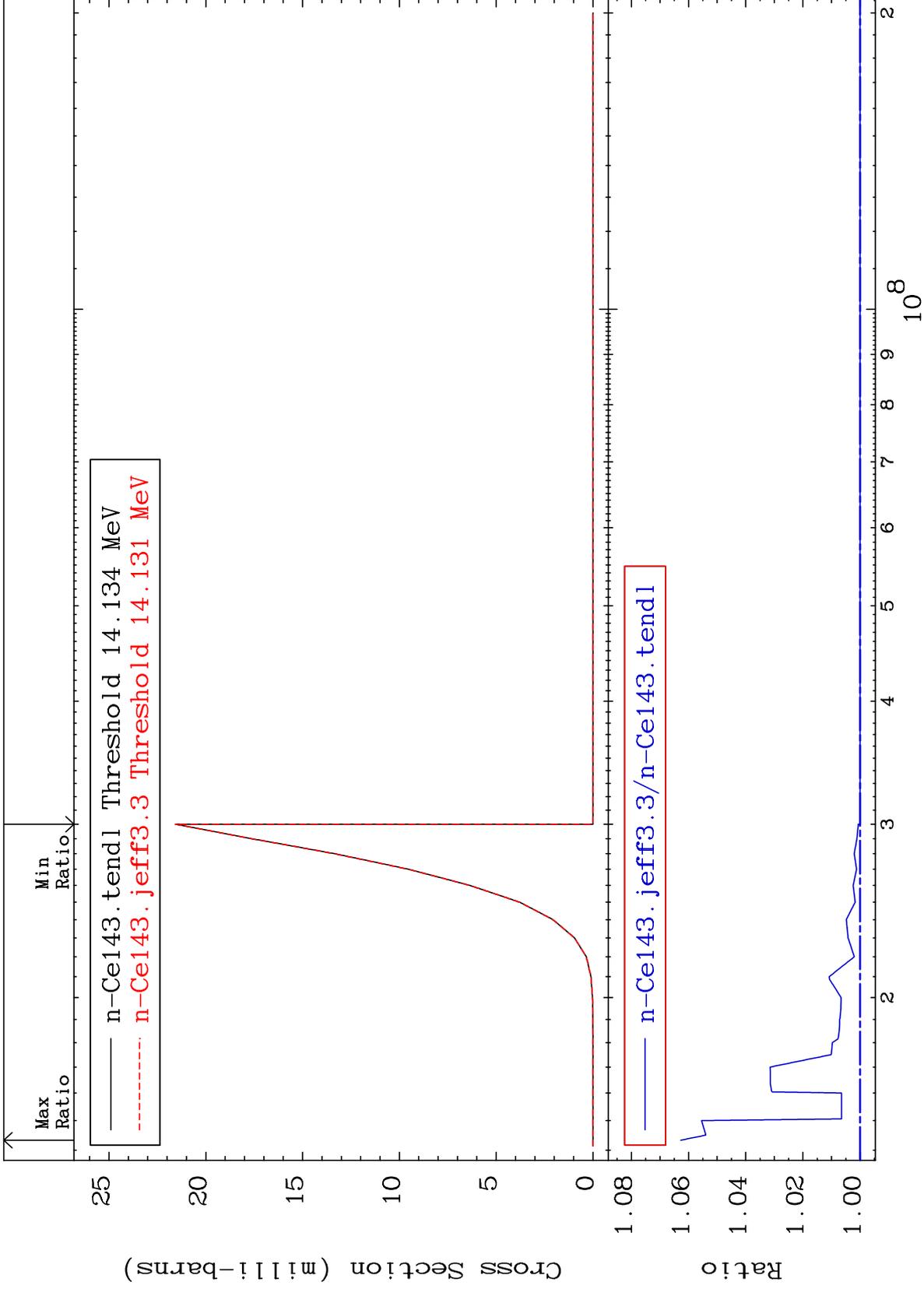
14

58-Ce-143

MAT 5846

(n,2n) p
Cross Section

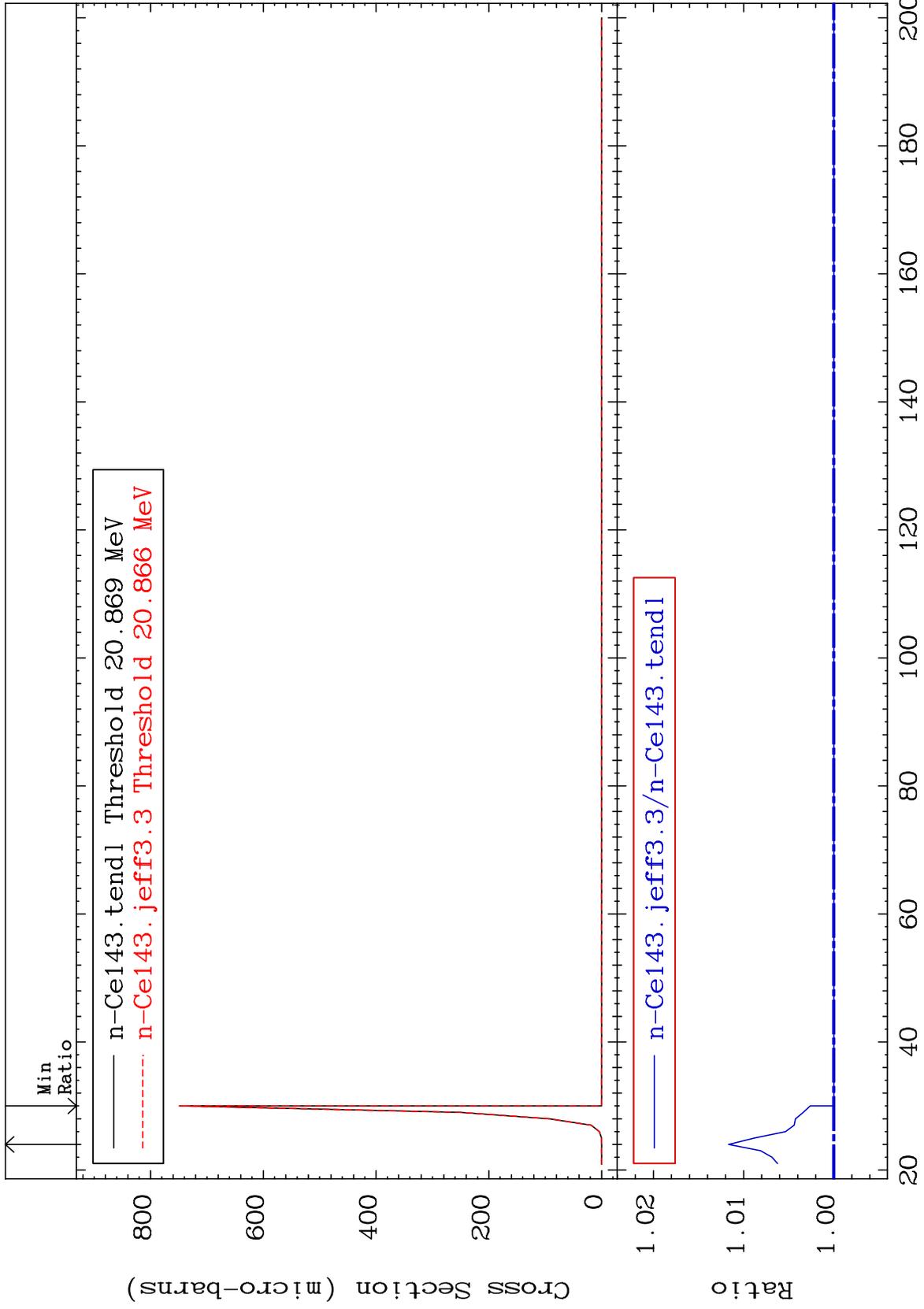
58-Ce-143
To 6.272 %



MAT 5846

(n,3n) p
Cross Section

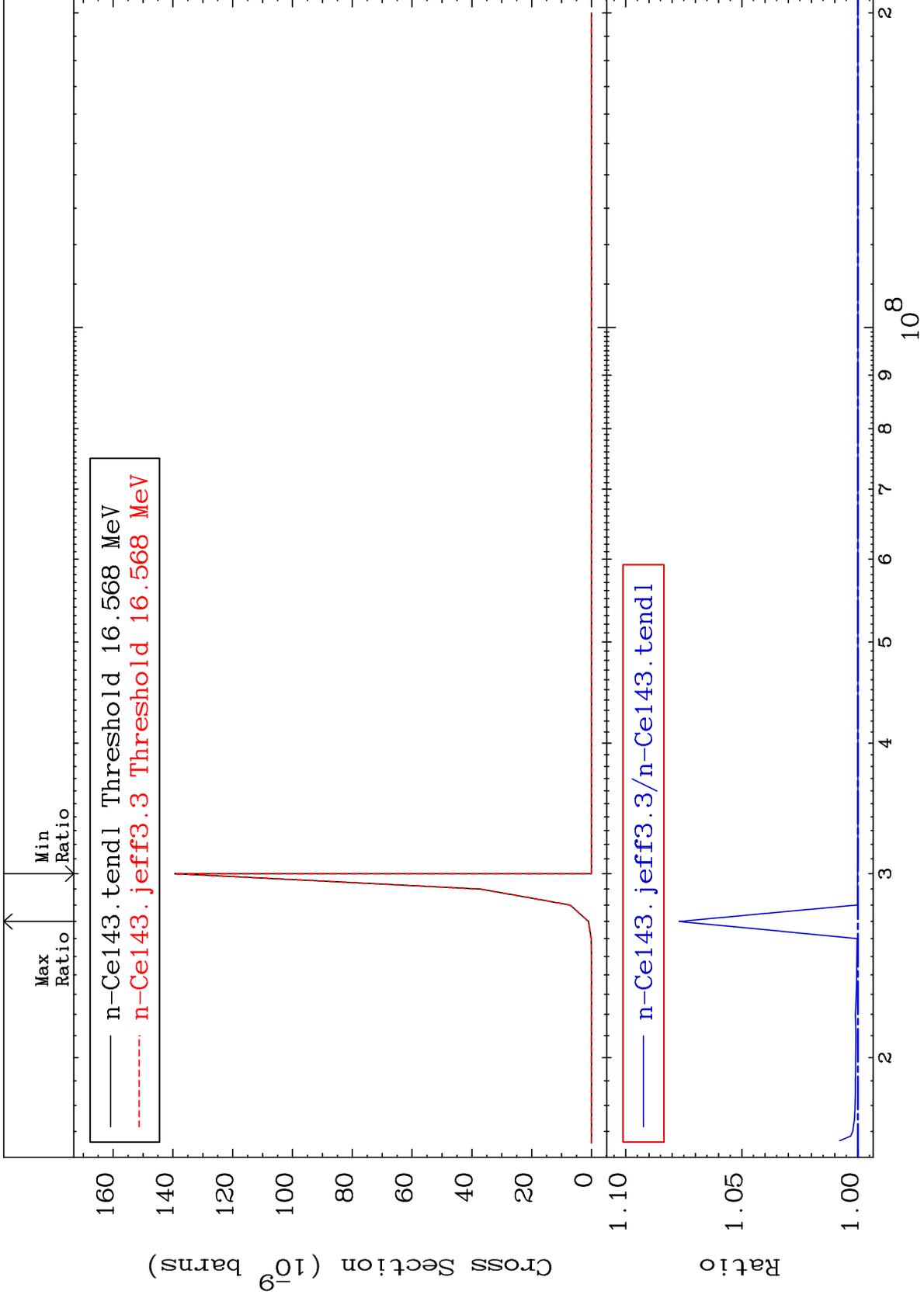
58-Ce-143
To 1.166 %



MAT 5846

(n,2n) p
Cross Section

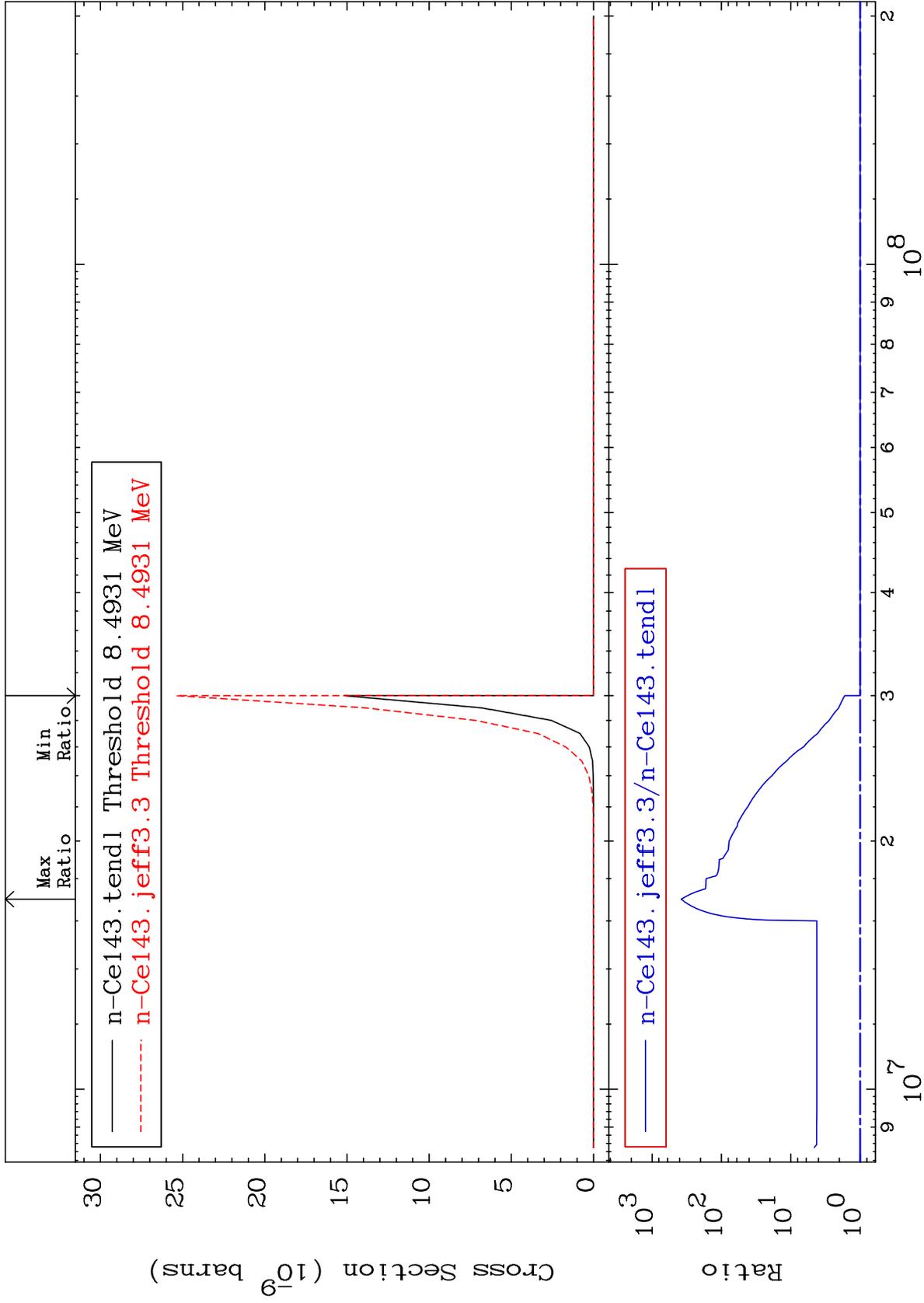
58-Ce-143
To 7.703 %



MAT 5846

(n,n') p α
Cross Section

58-Ce-143
To 9999. %
0.000



18

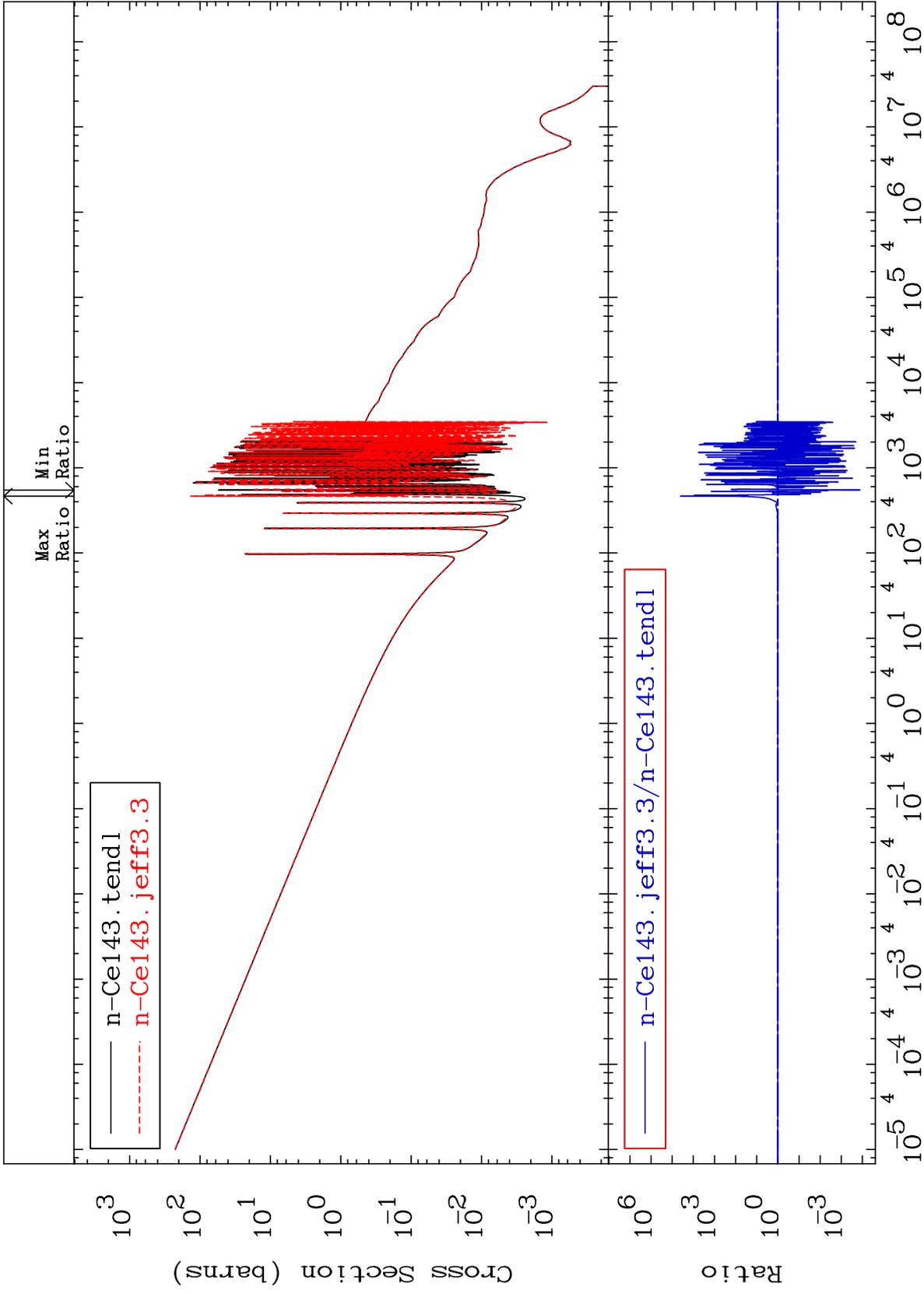
Incident Energy (eV)

58-Ce-143

MAT 5846

(n, γ)
Cross Section

58-Ce-143
-99.99 To 9999. %



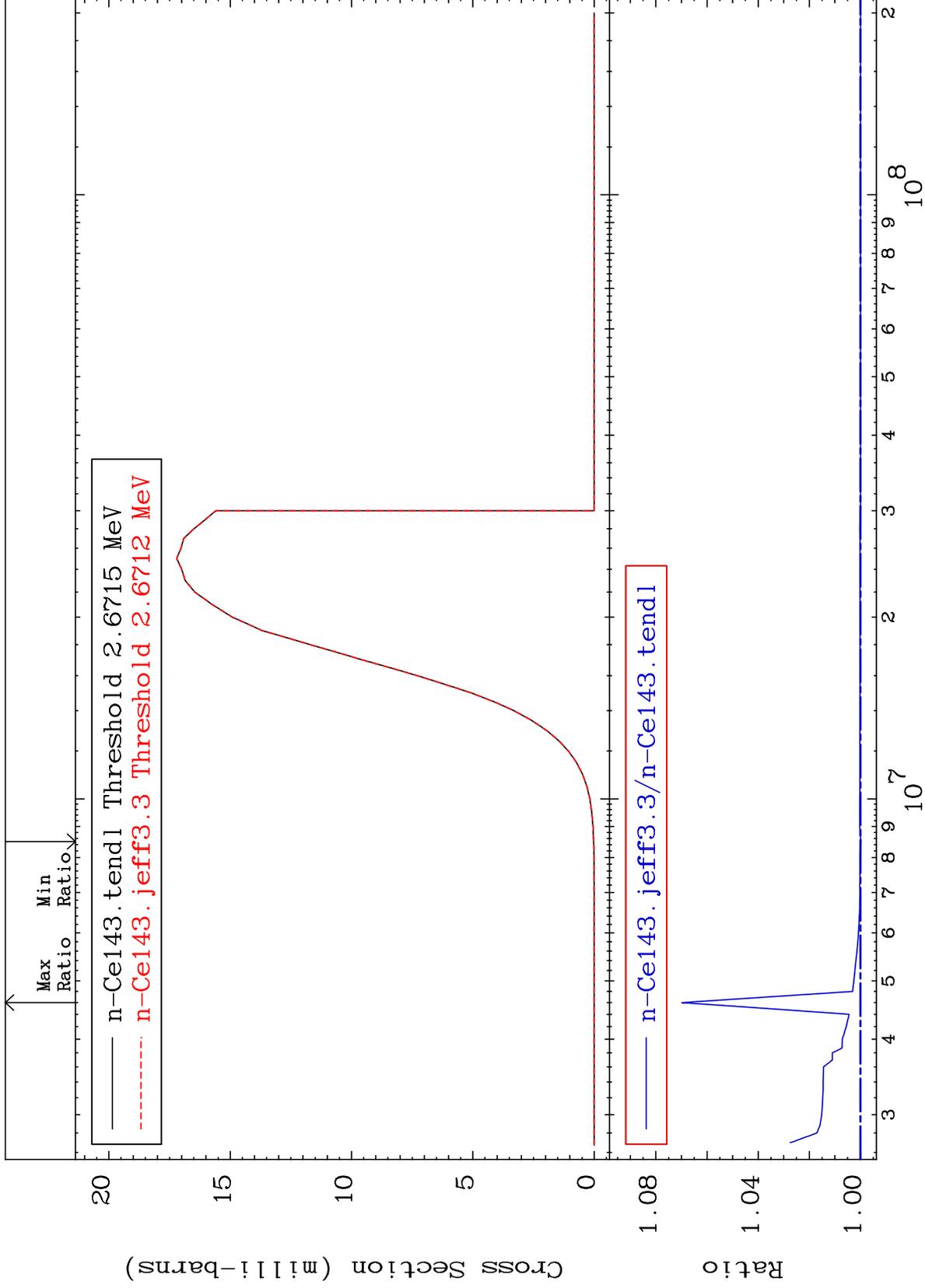
MAT 5846

(n,p)

58-Ce-143

Cross Section

-0.028 To 6.982 %



20

Incident Energy (eV)

58-Ce-143

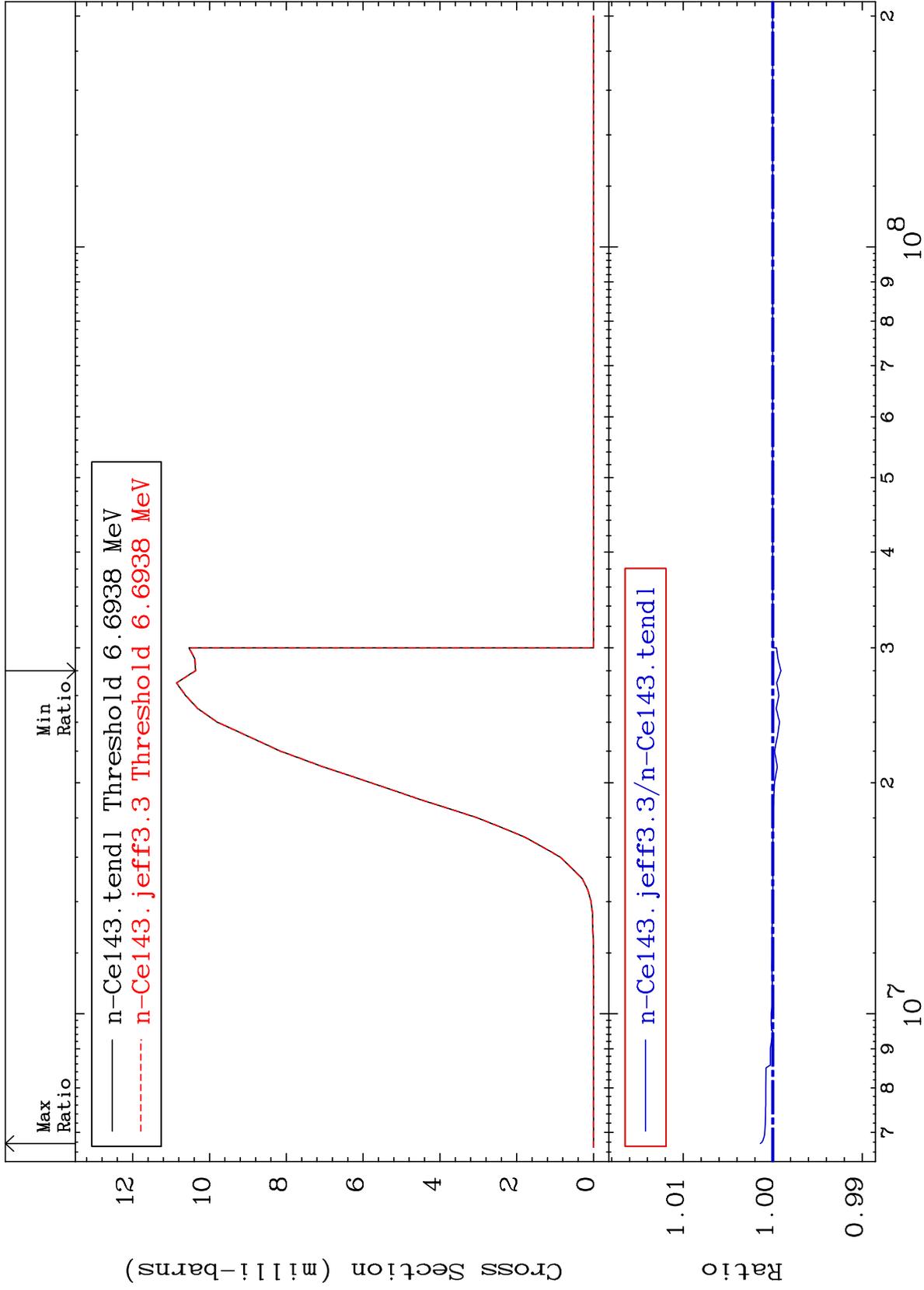
MAT 5846

(n, d)

58-Ce-143

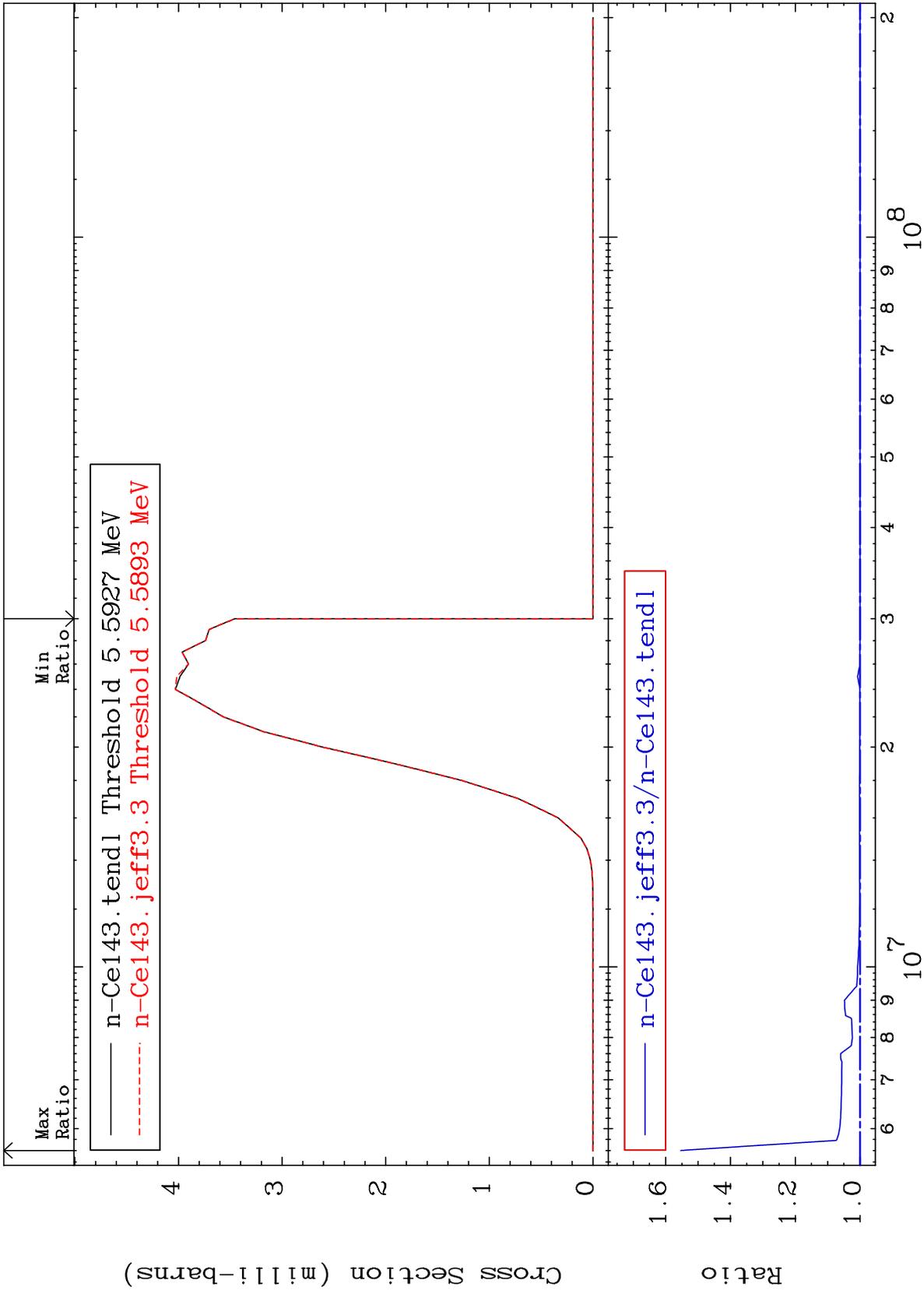
Cross Section

-0.092 To 0.137 %



MAT 5846

(n, t)
Cross Section
58-Ce-143
To 55.34 %
0.000



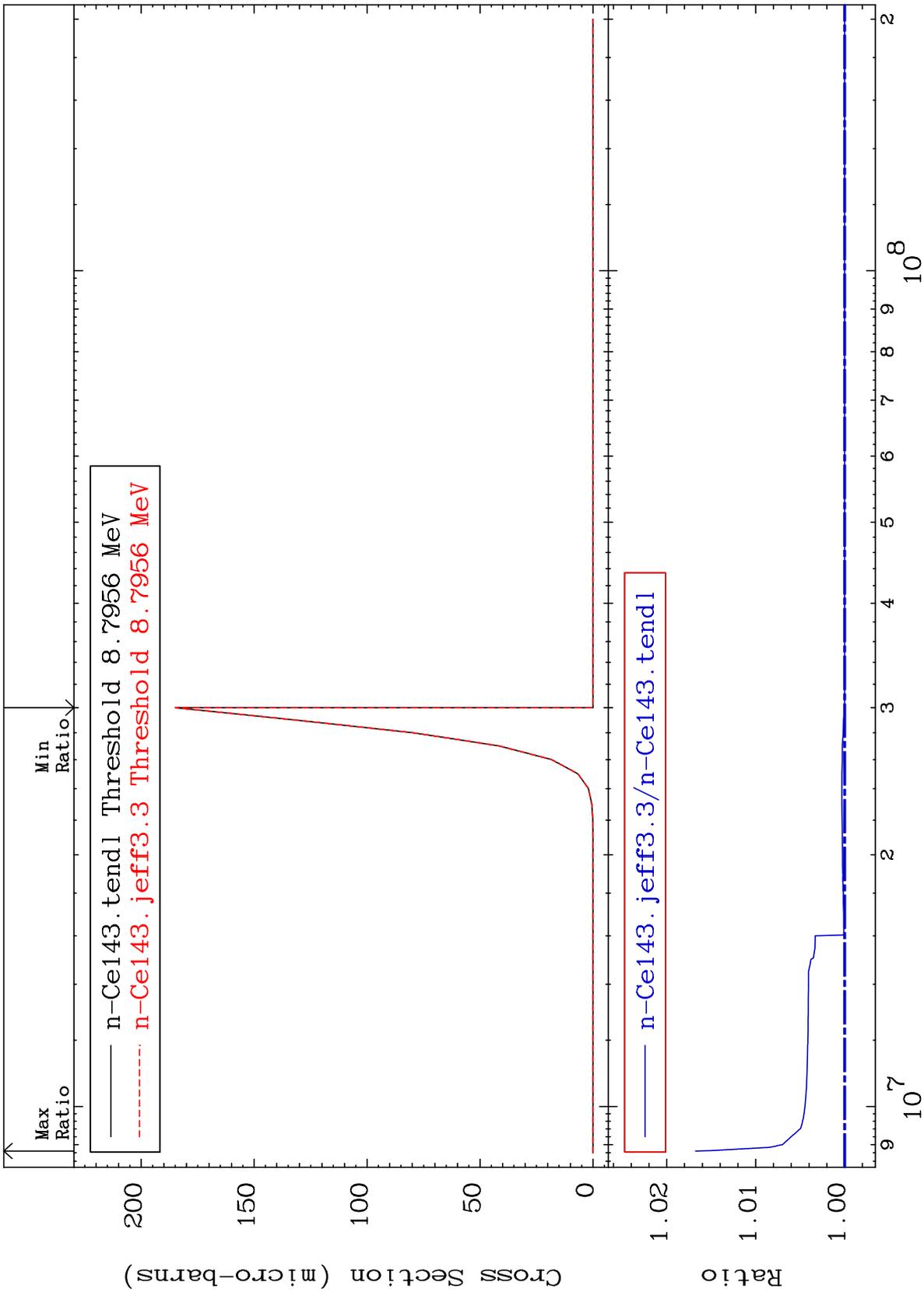
MAT 5846

(n, He-3)

58-Ce-143

Cross Section

-0.007 To 1.676 %



23

58-Ce-143

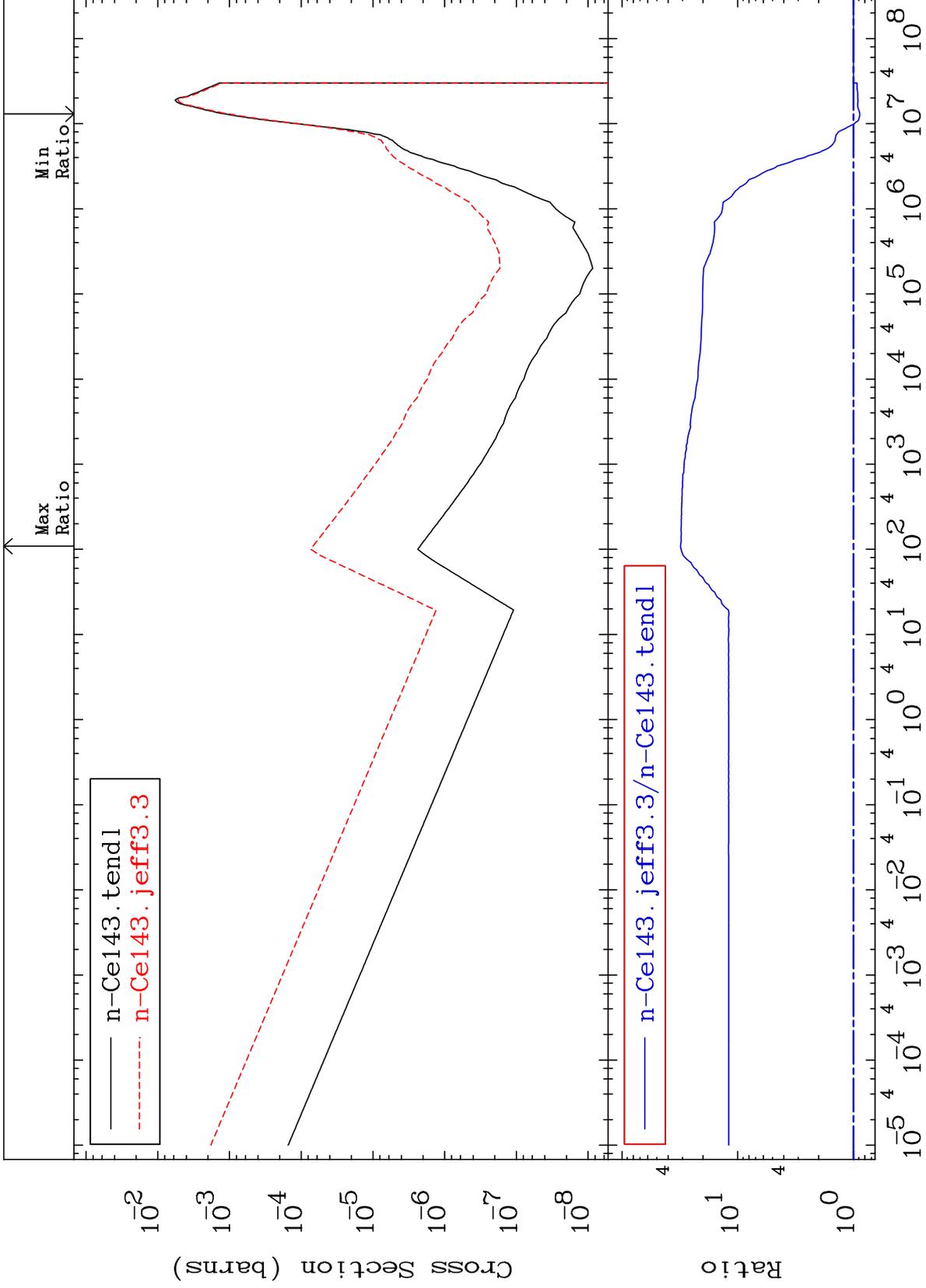
MAT 5846

(n, α)

58-Ce-143

Cross Section

-11.71 To 3022. %



24

Incident Energy (eV)

58-Ce-143

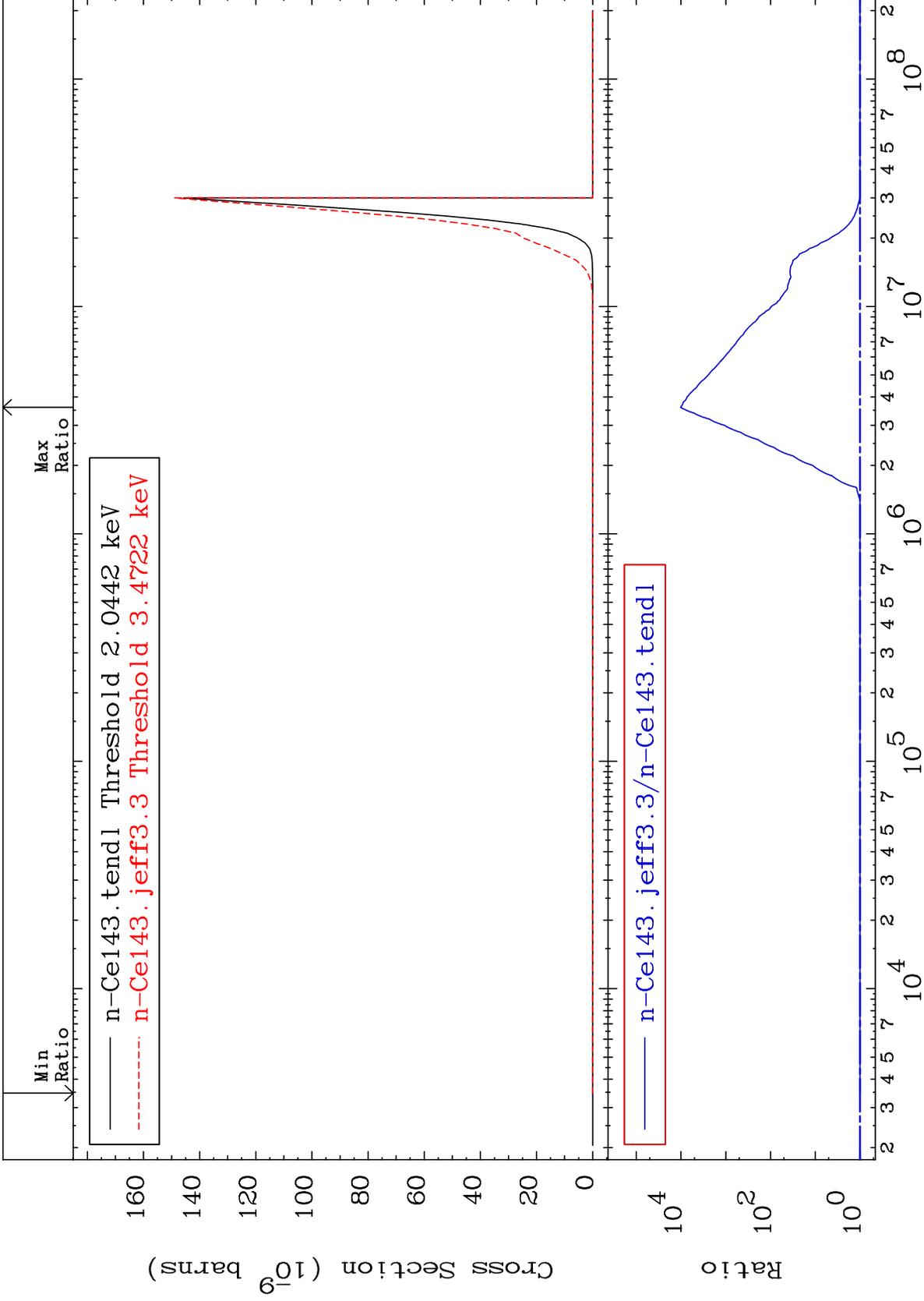
MAT 5846

(n,2α)

58-Ce-143

Cross Section

0.000 To 9999. %



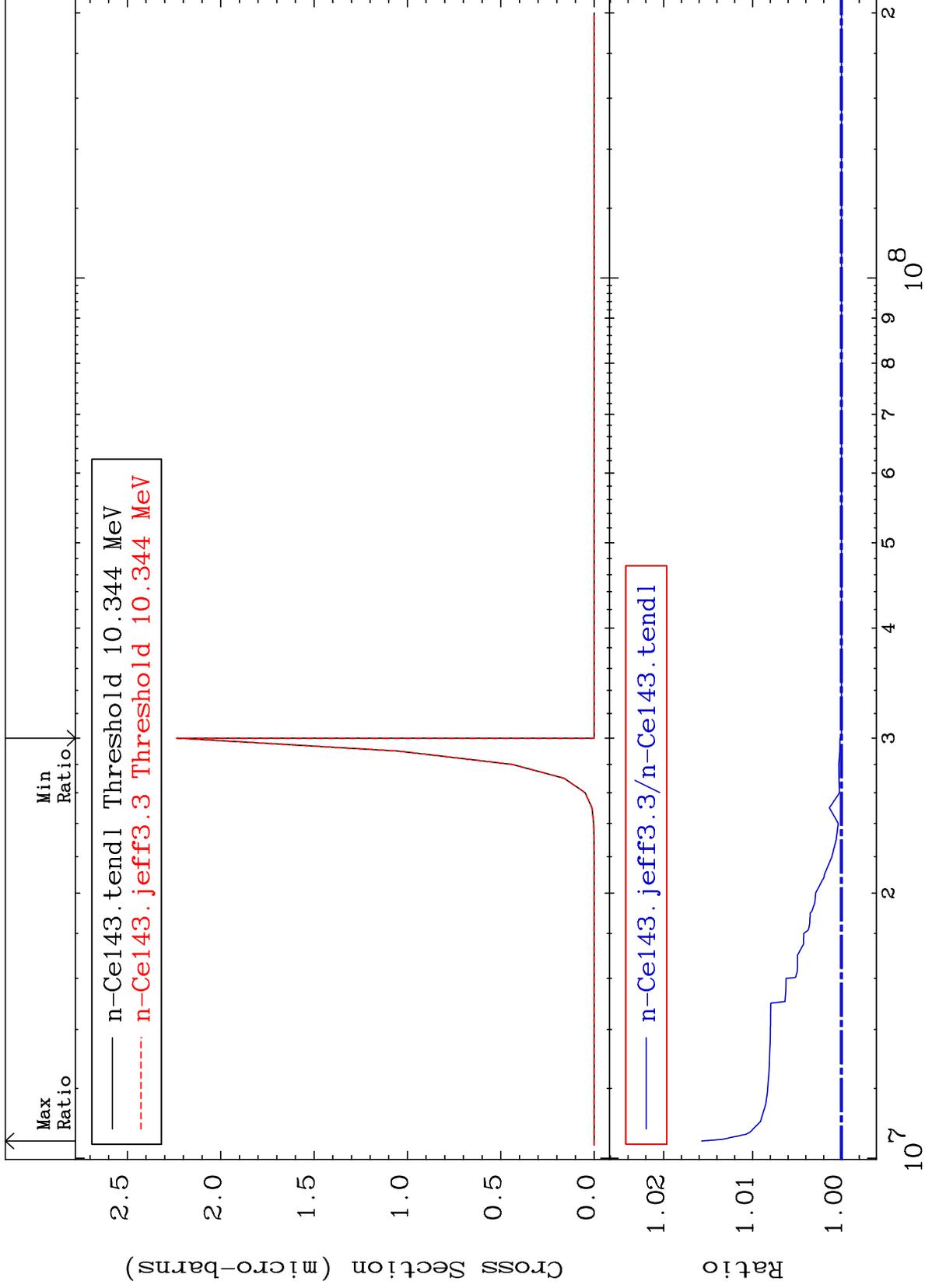
MAT 5846

(n,2p)

58-Ce-143

Cross Section

0.000 To 1.573 %



26

58-Ce-143

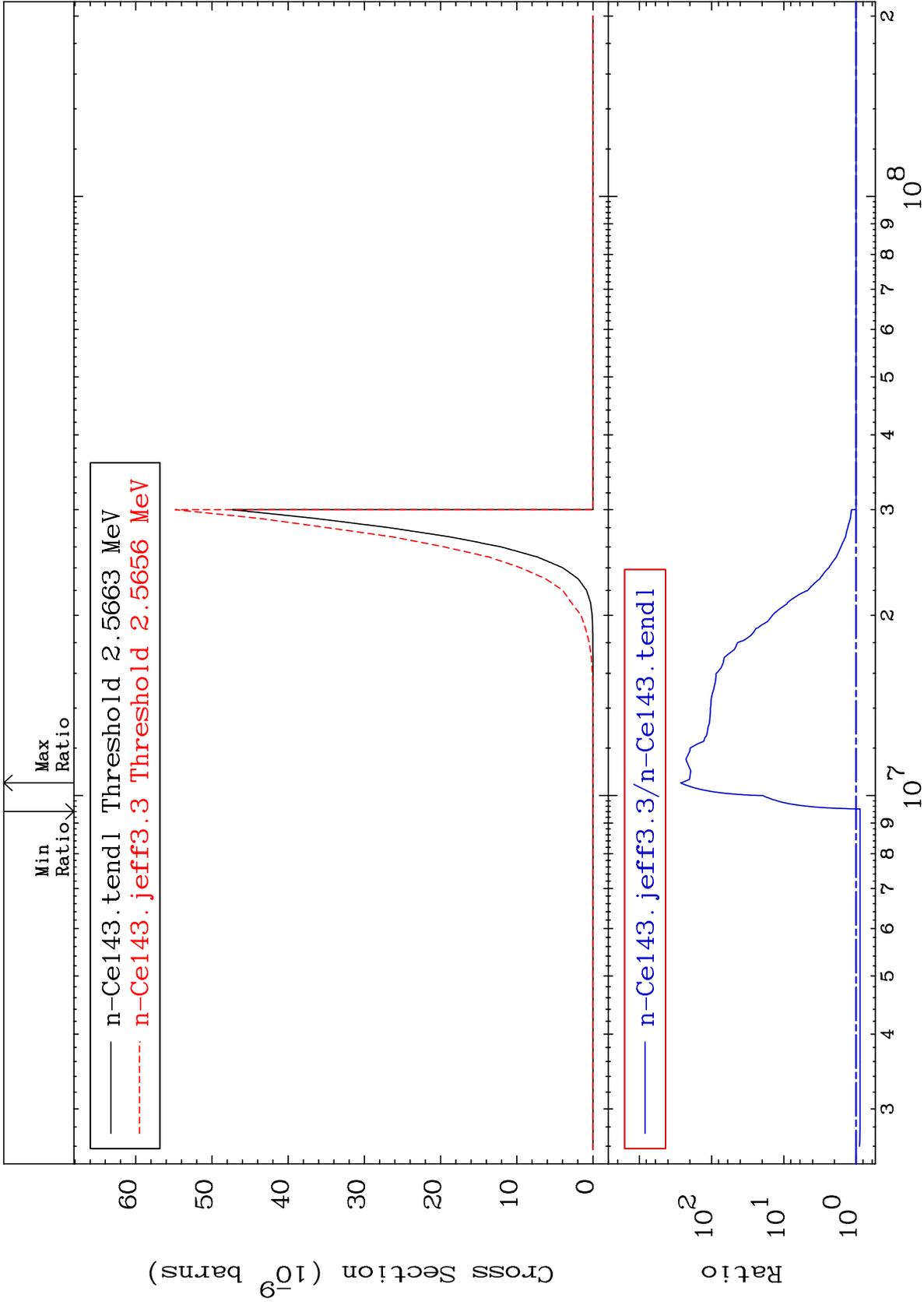
MAT 5846

(n, p) α

58-Ce-143

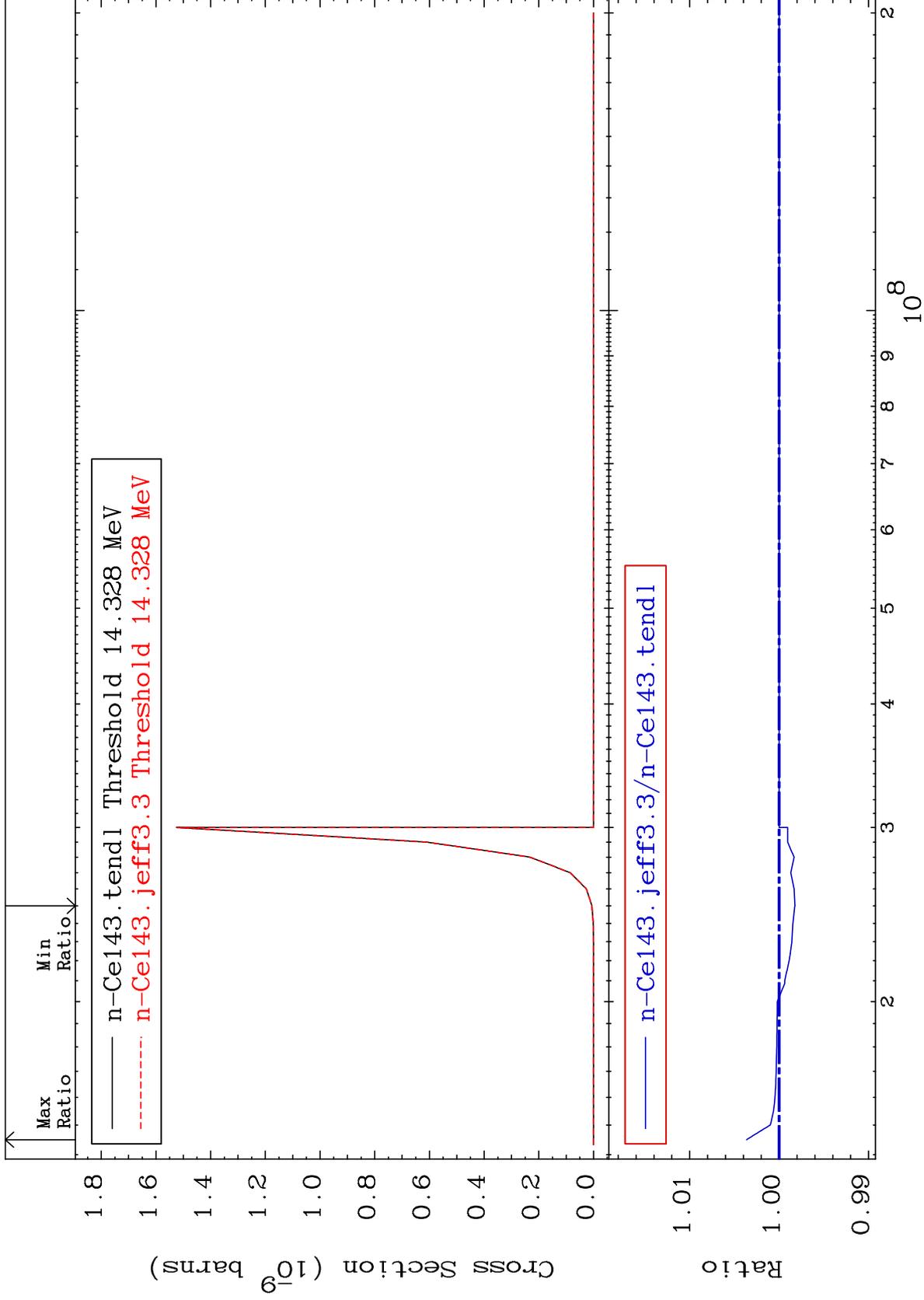
Cross Section

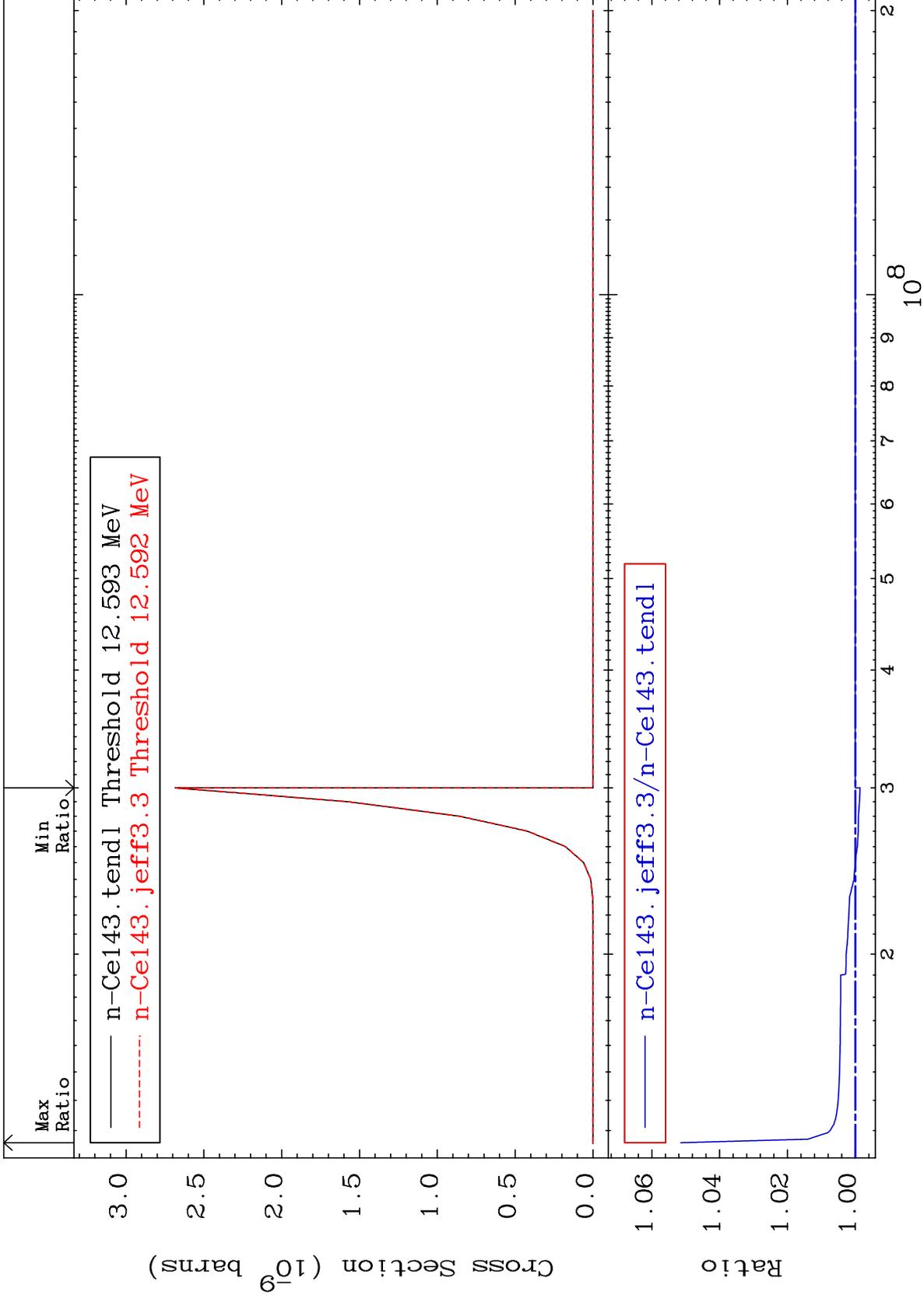
-11.96 To 9999. %



Cross Section

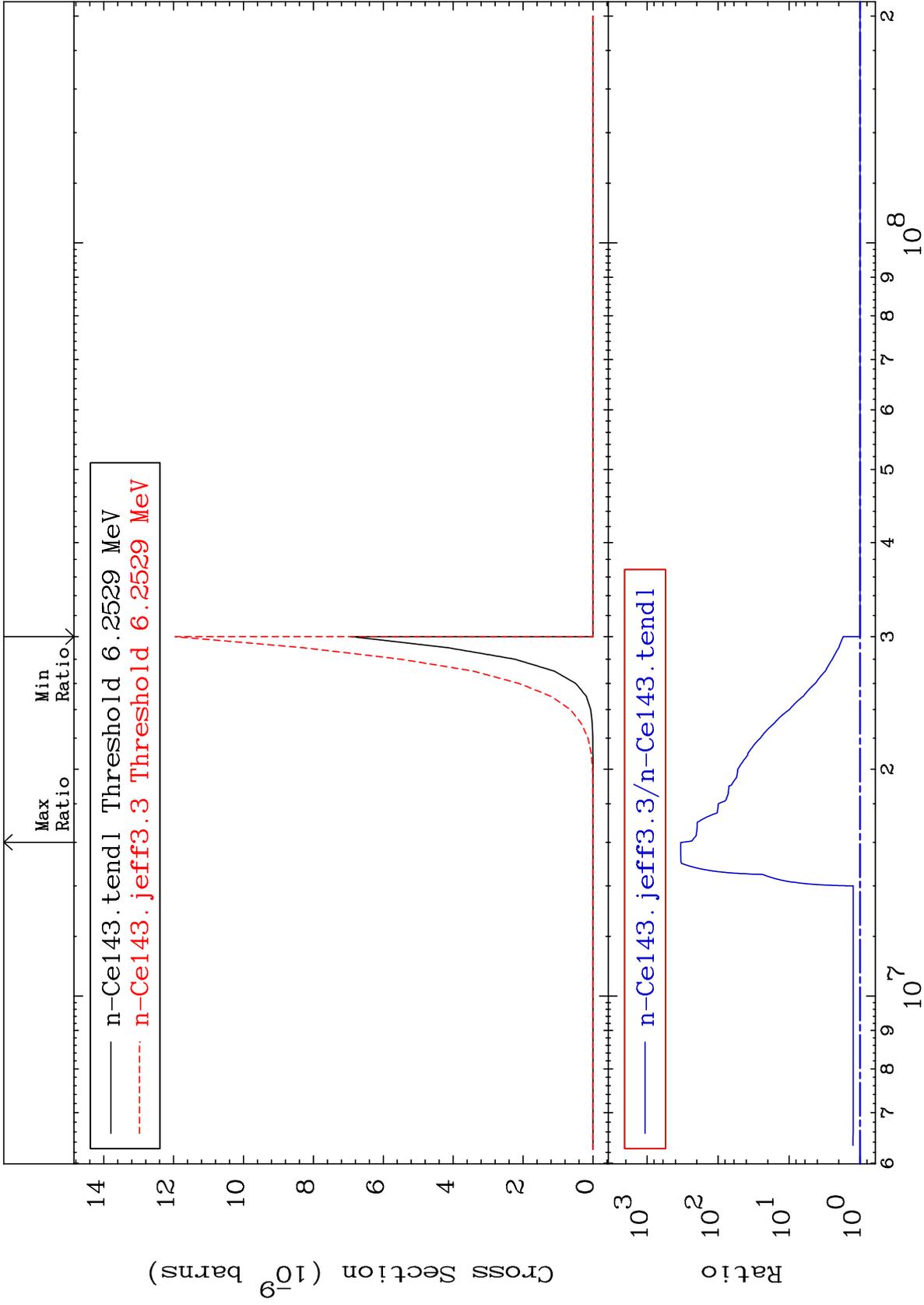
-0.176 To 0.365 %





MAT 5846

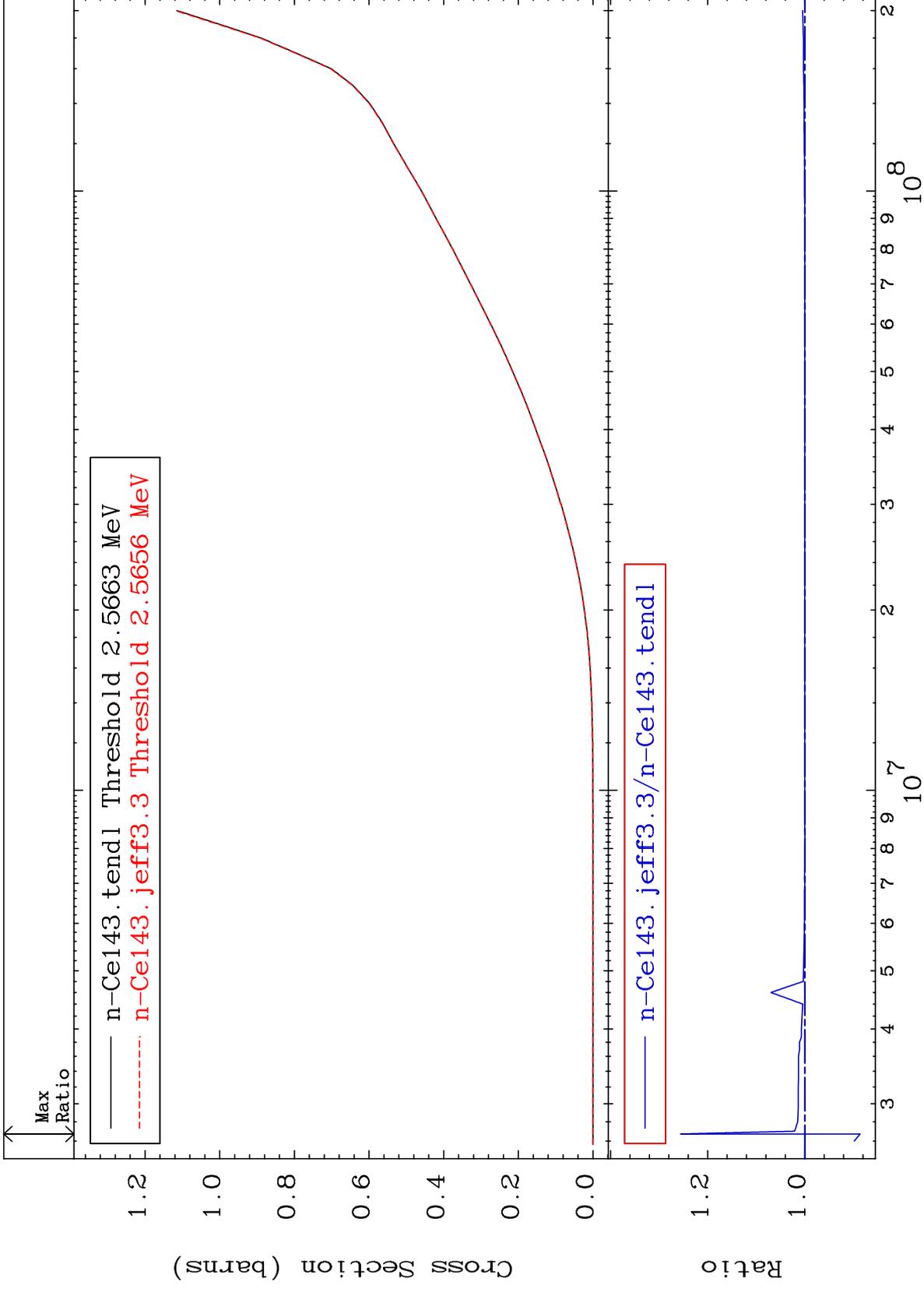
(n, d) α Cross Section
58-Ce-143
To 9999. %
0.000



30

Incident Energy (eV)

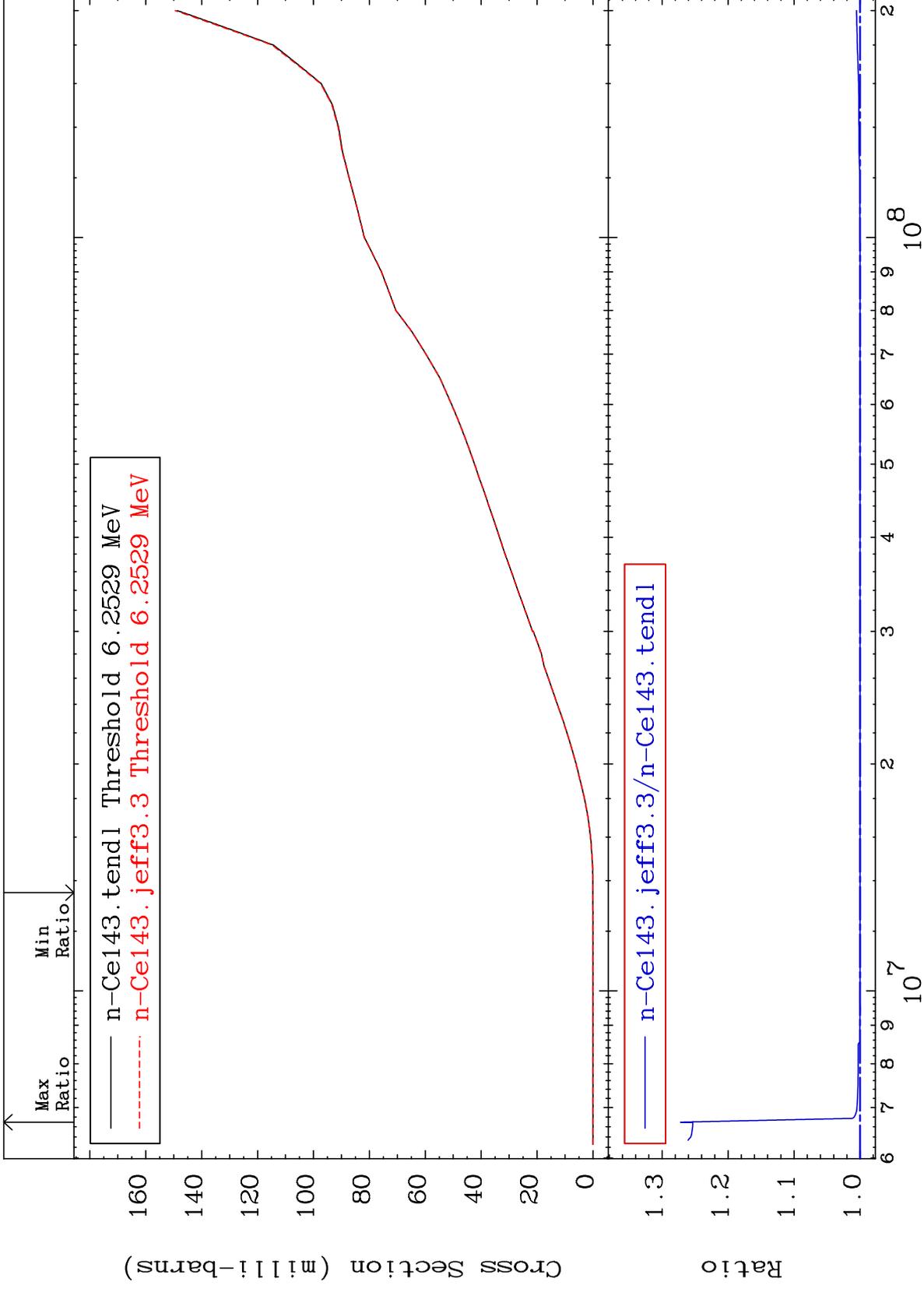
58-Ce-143



MAT 5846

Deuterium Production
Cross Section

58-Ce-143
-0.002 To 27.16 %



32

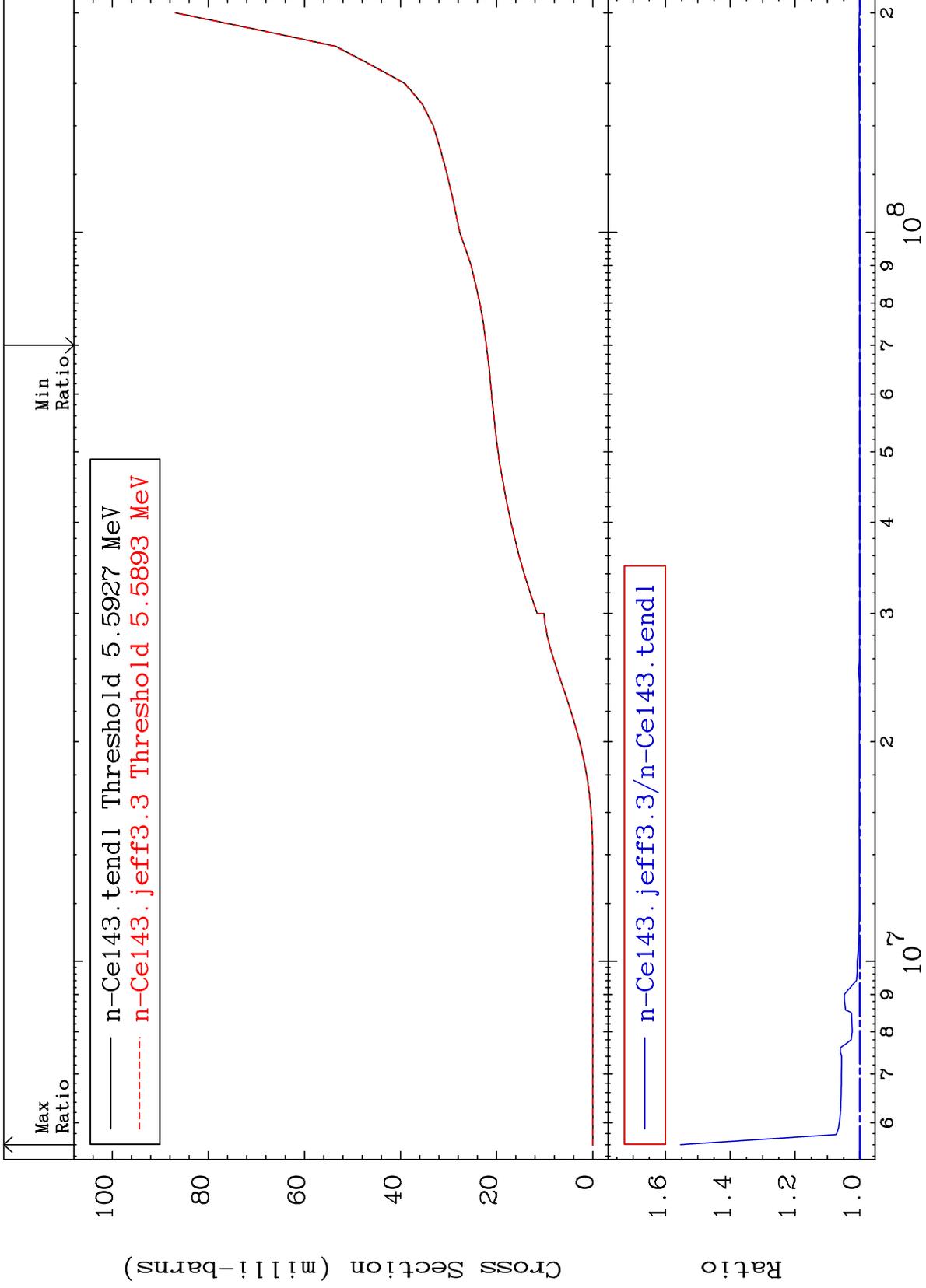
Incident Energy (eV)

58-Ce-143

MAT 5846

Tritium Production
Cross Section

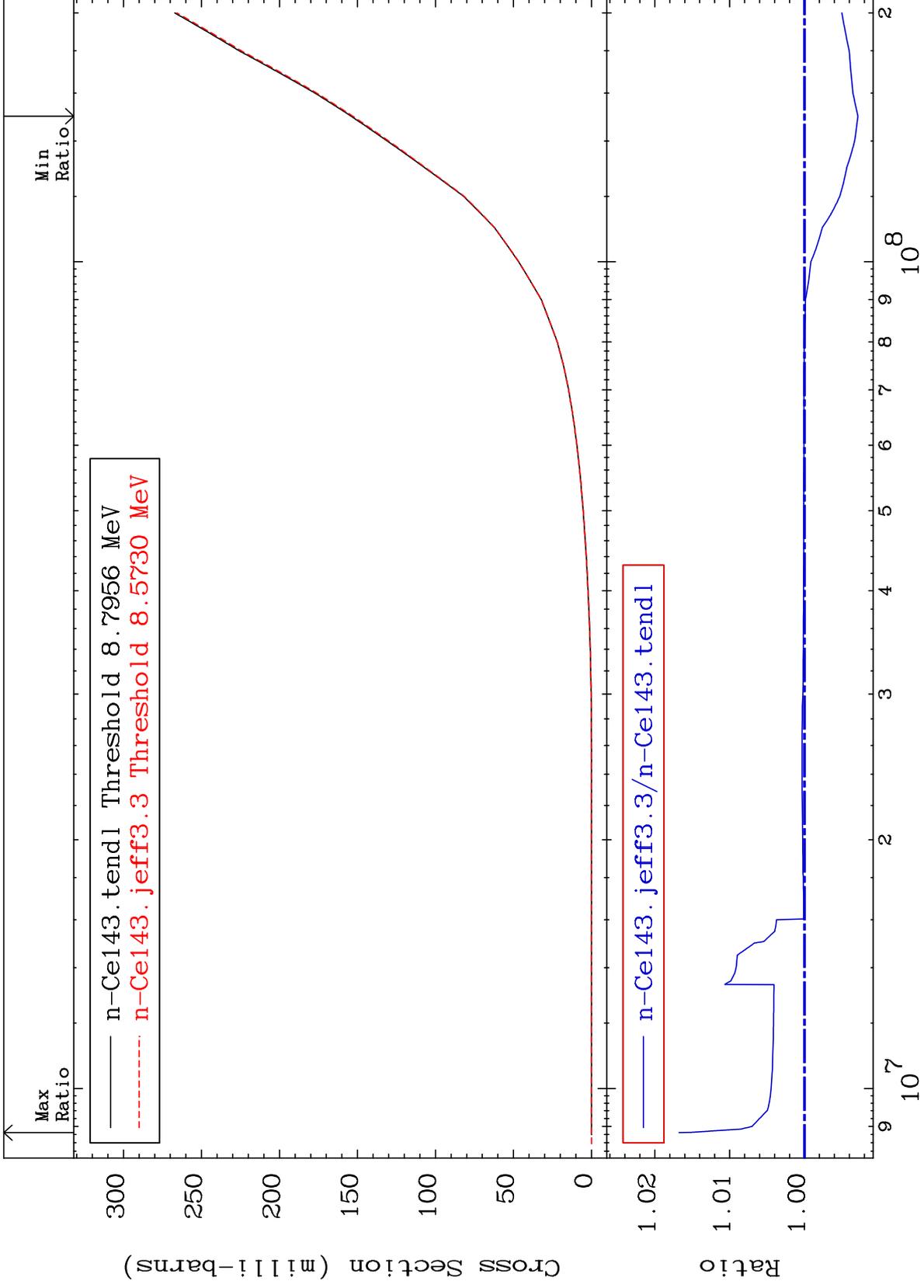
58-Ce-143
0.028 To 55.34 %



MAT 5846

He-3 Production
Cross Section

58-Ce-143
-0.714 To 1.676 %



34

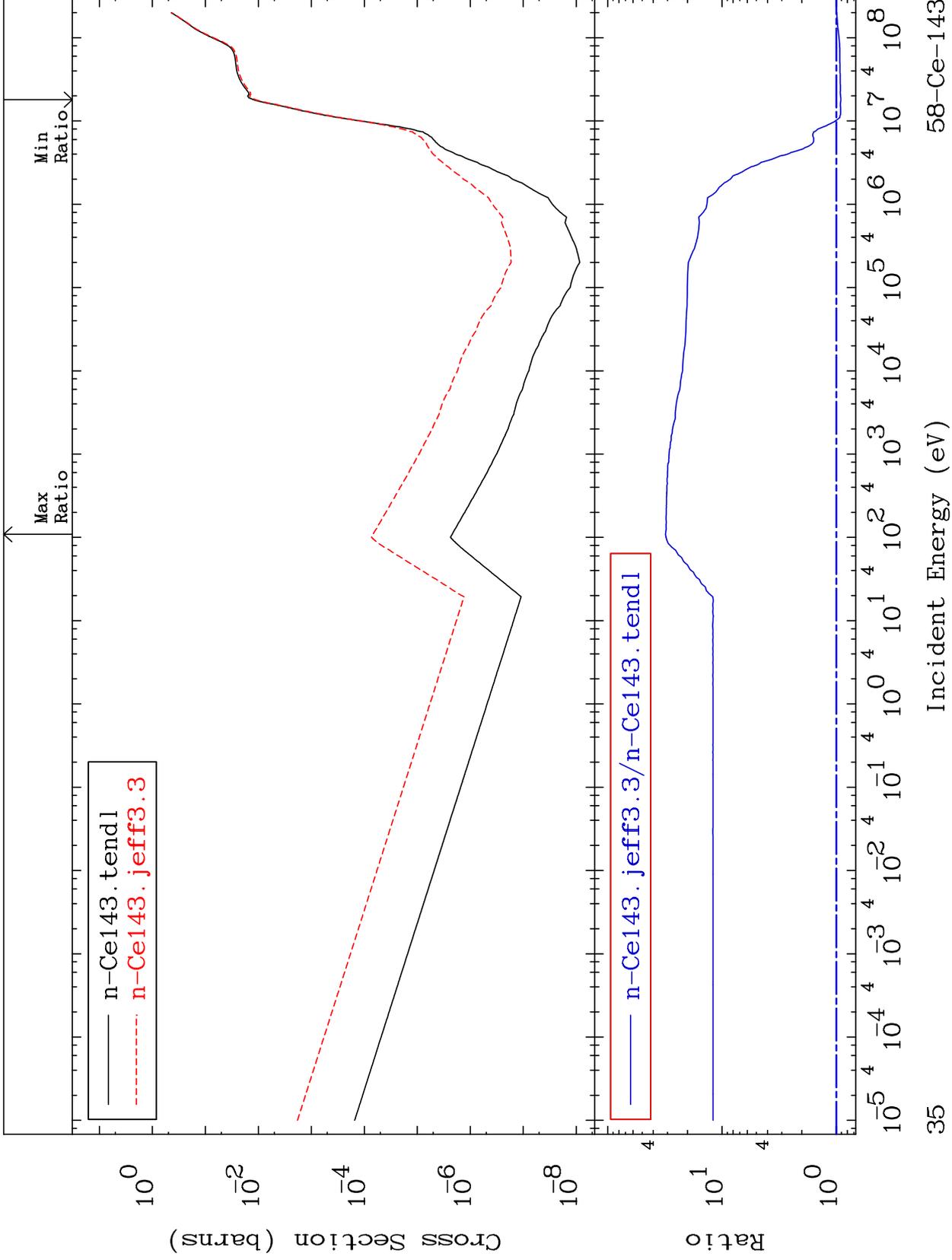
Incident Energy (eV)

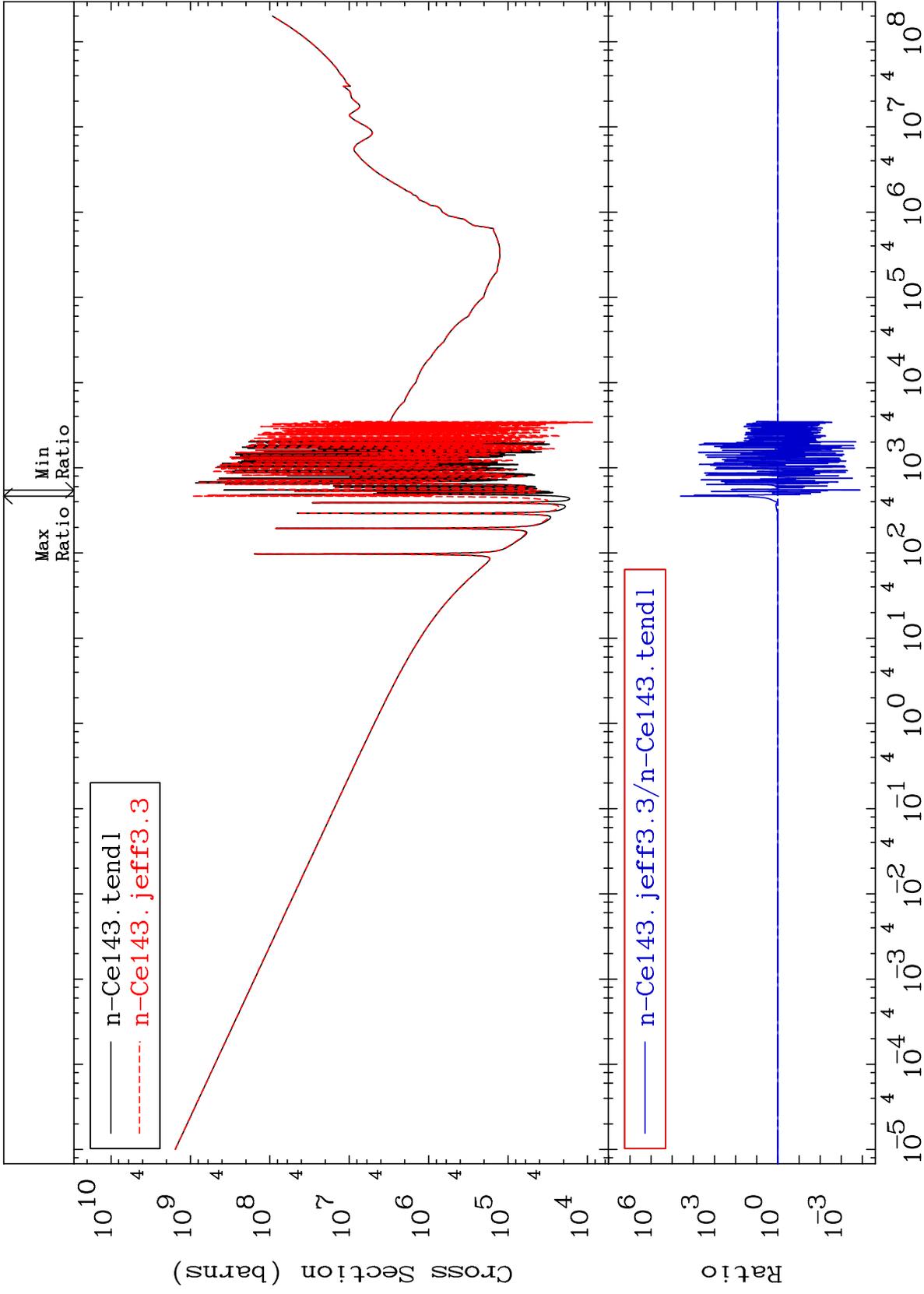
58-Ce-143

MAT 5846

He-4 Production
Cross Section

58-Ce-143
-8.813 To 3022. %

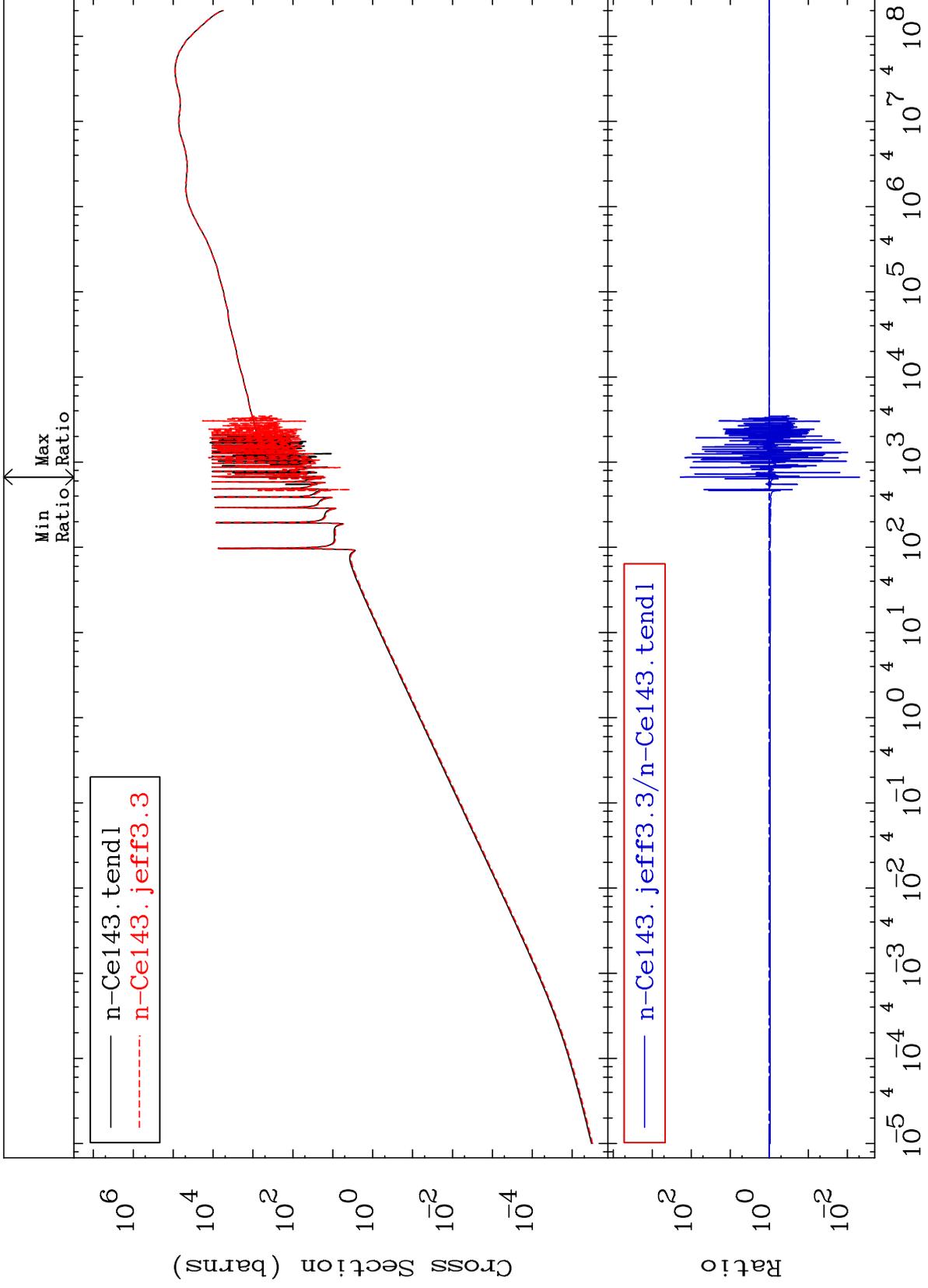




MAT 5846

Kerma elastic
Cross Section

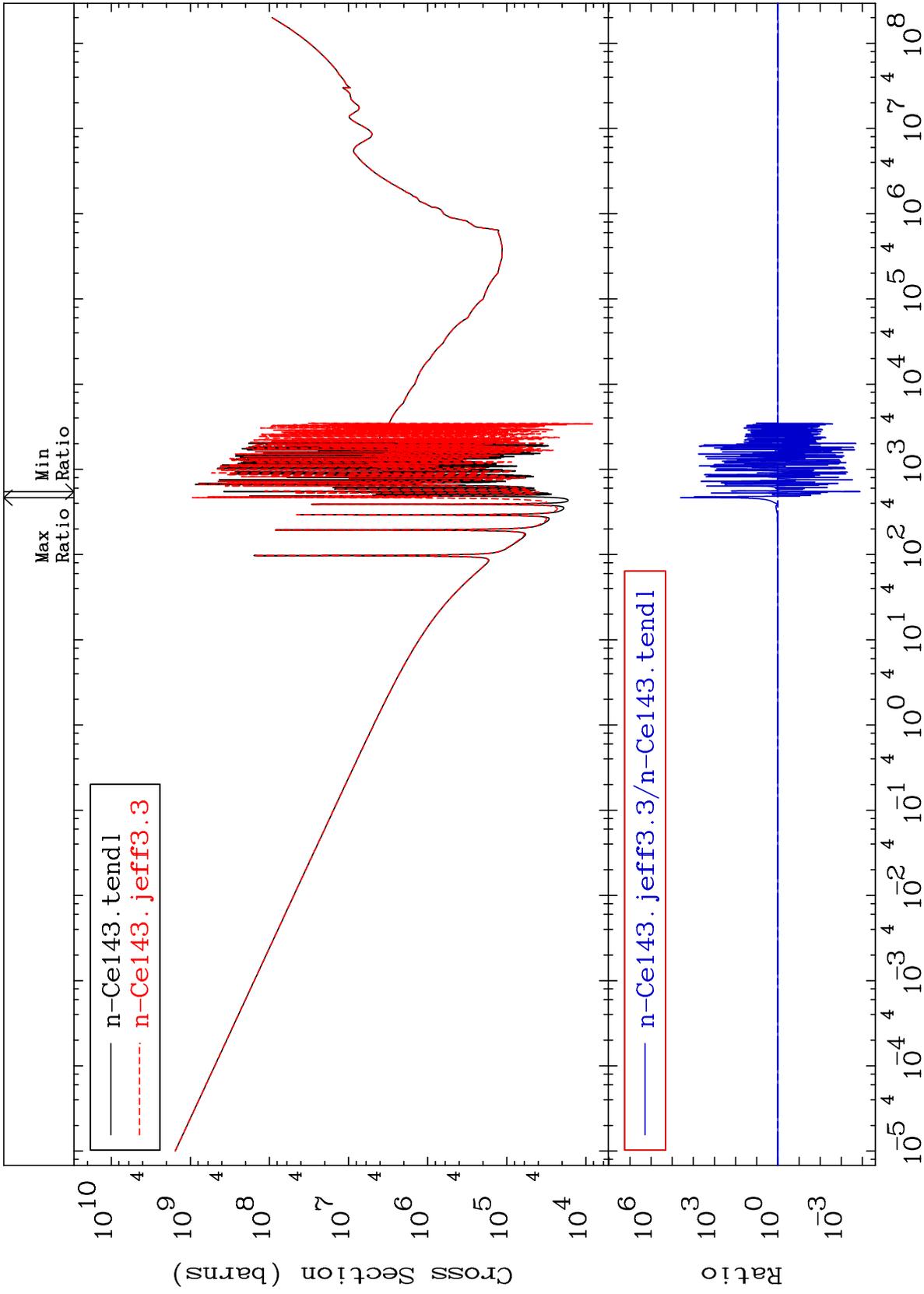
58-Ce-143
-99.51 To 9999. %

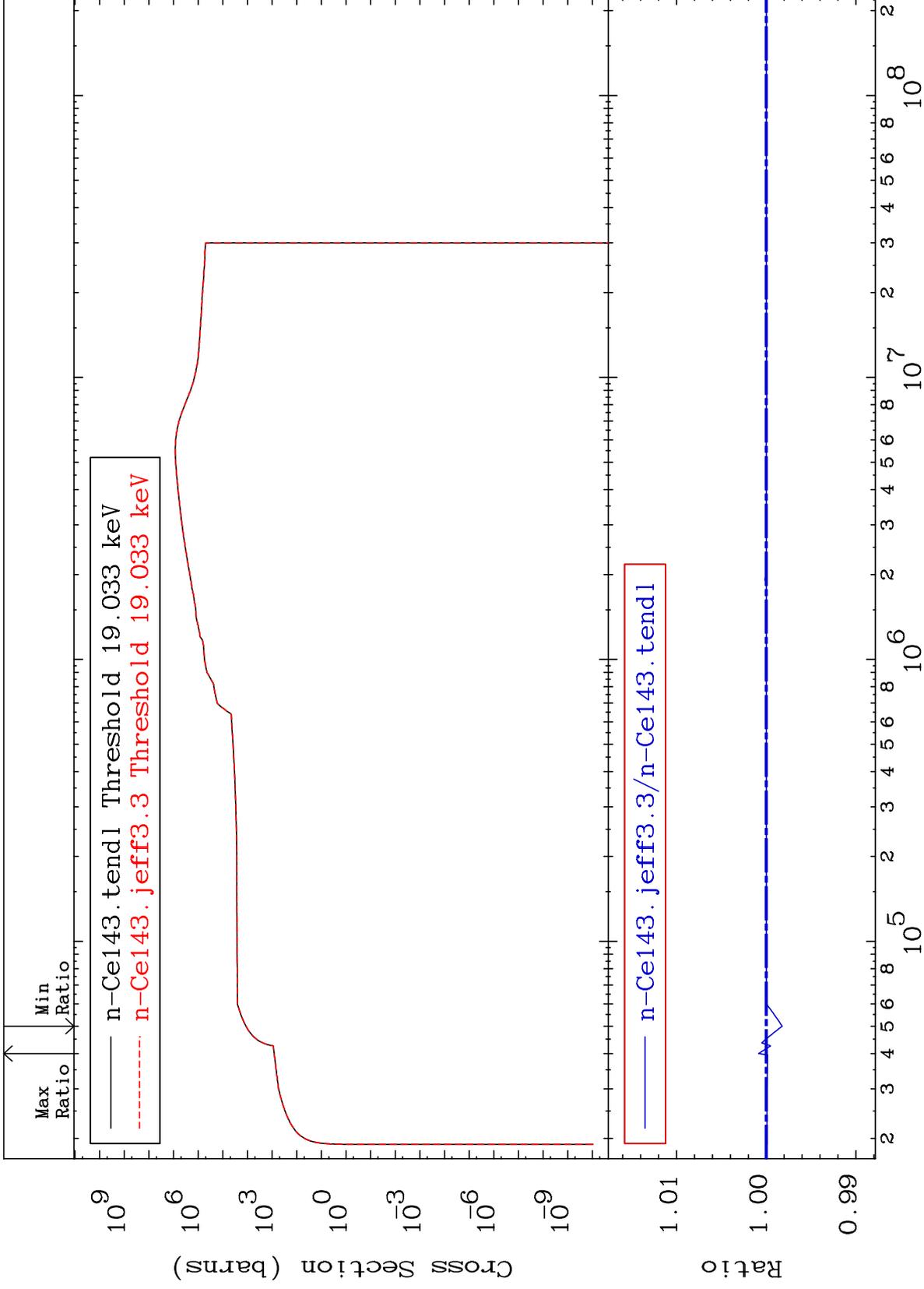


37

Incident Energy (eV)

58-Ce-143

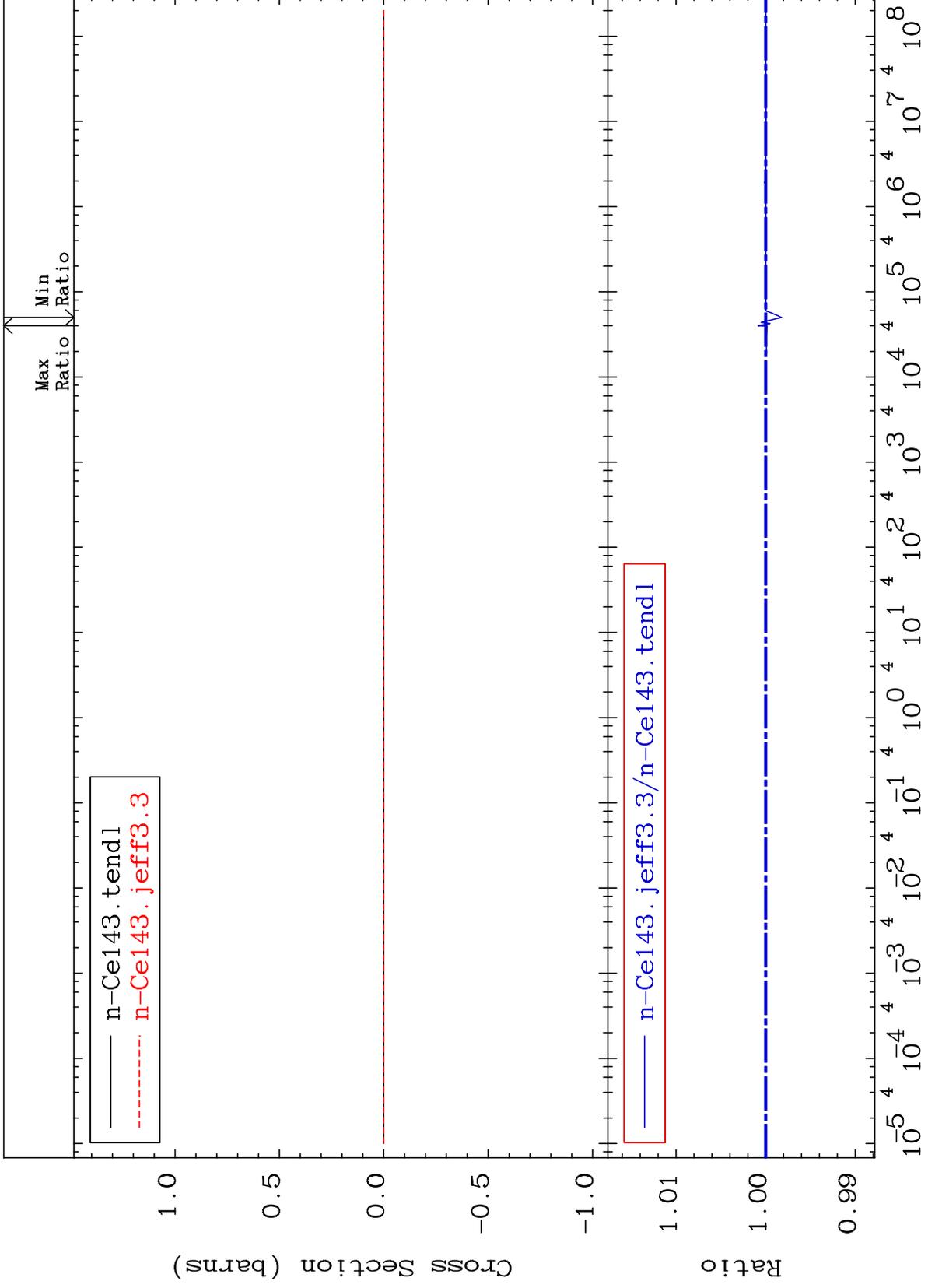




MAT 5846

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

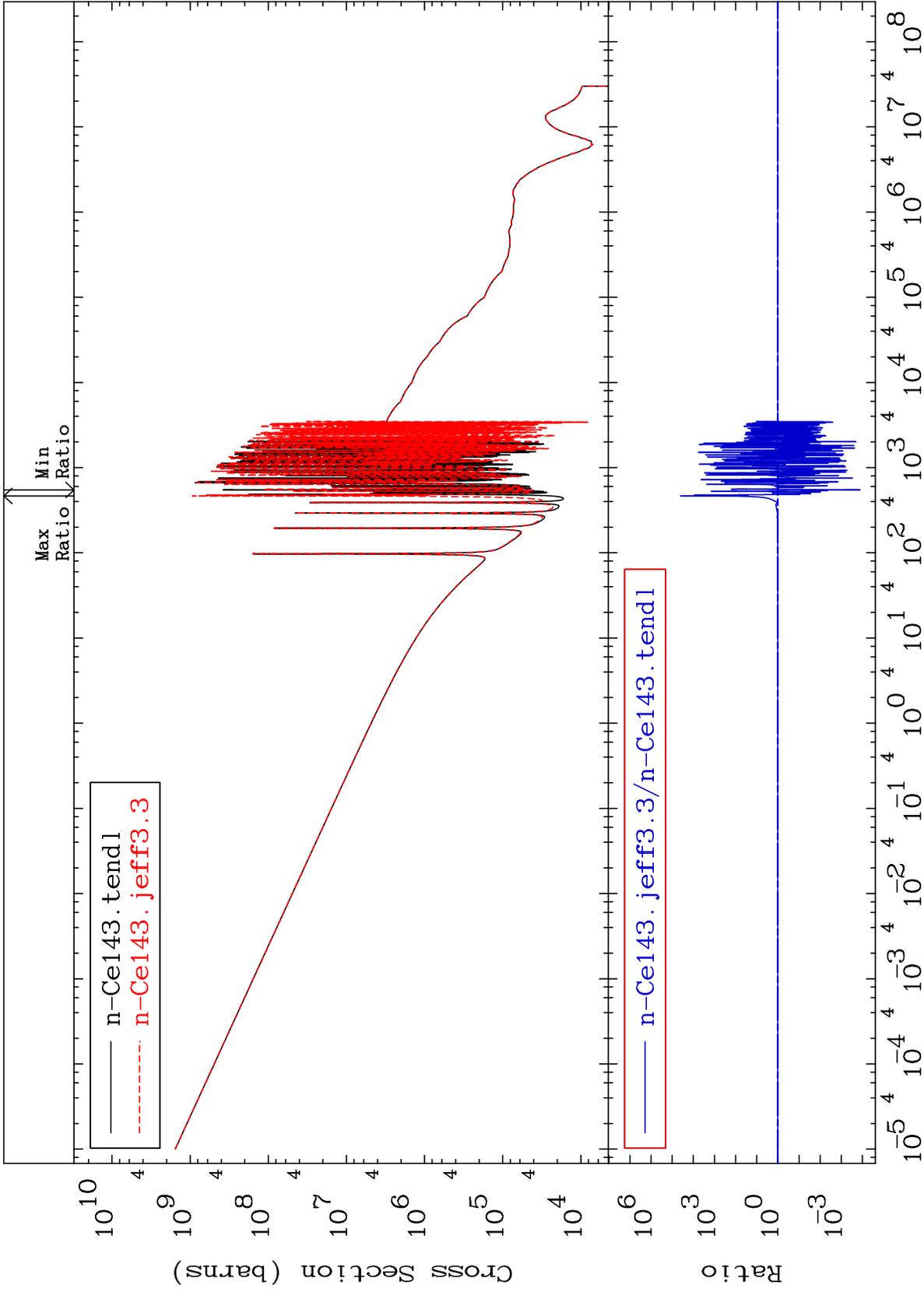
58-Ce-143
-0.178 To 0.084 %

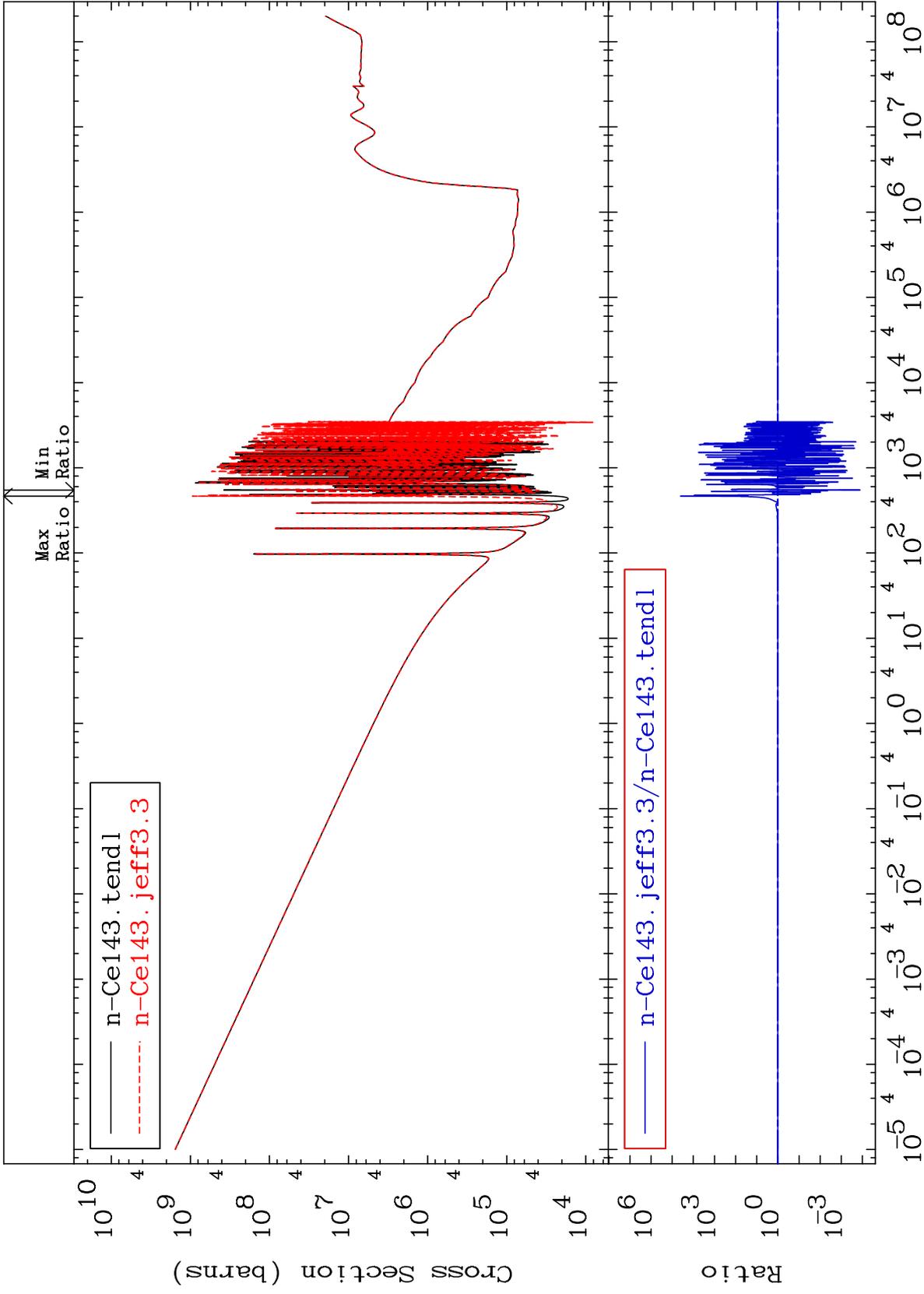


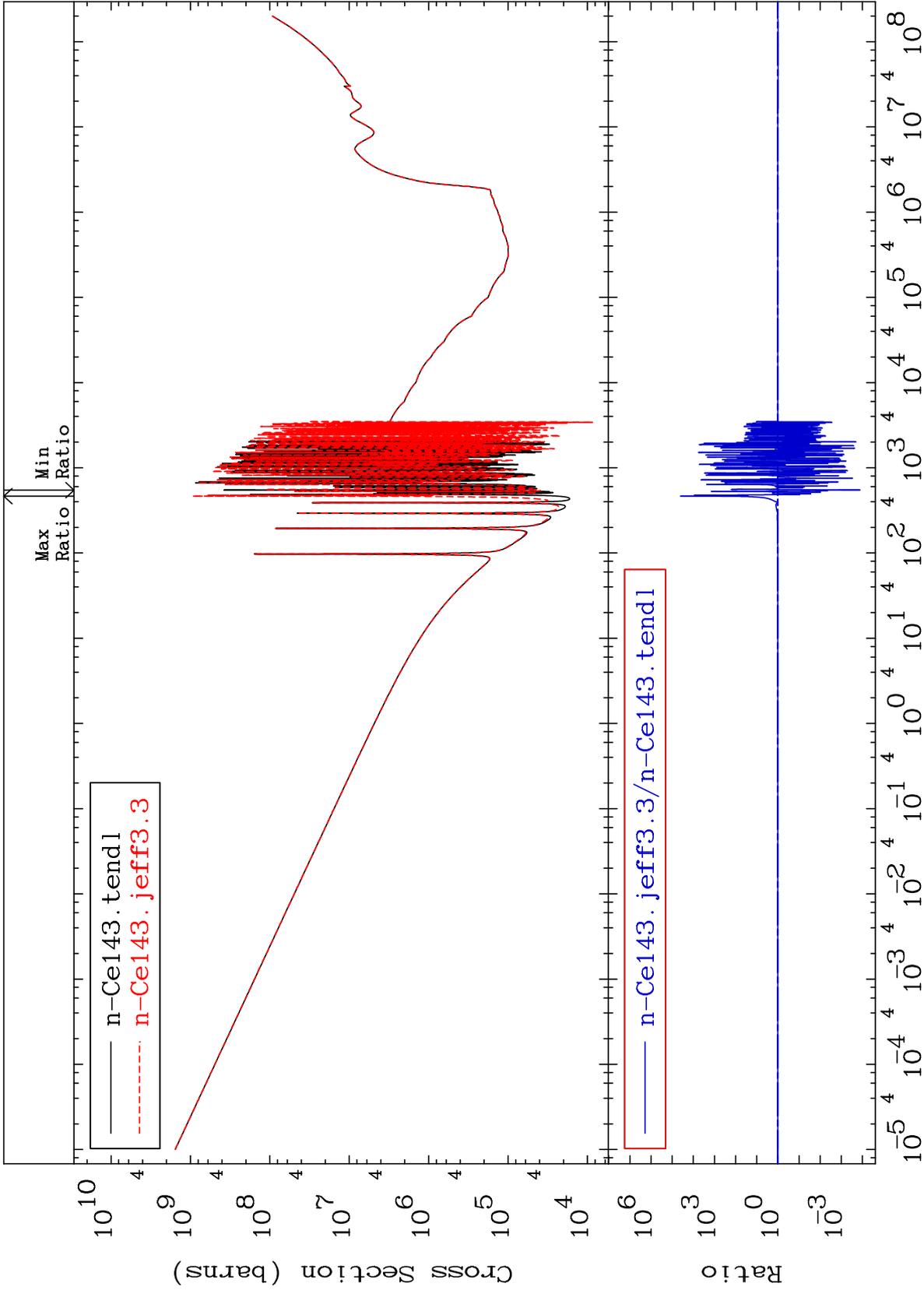
40

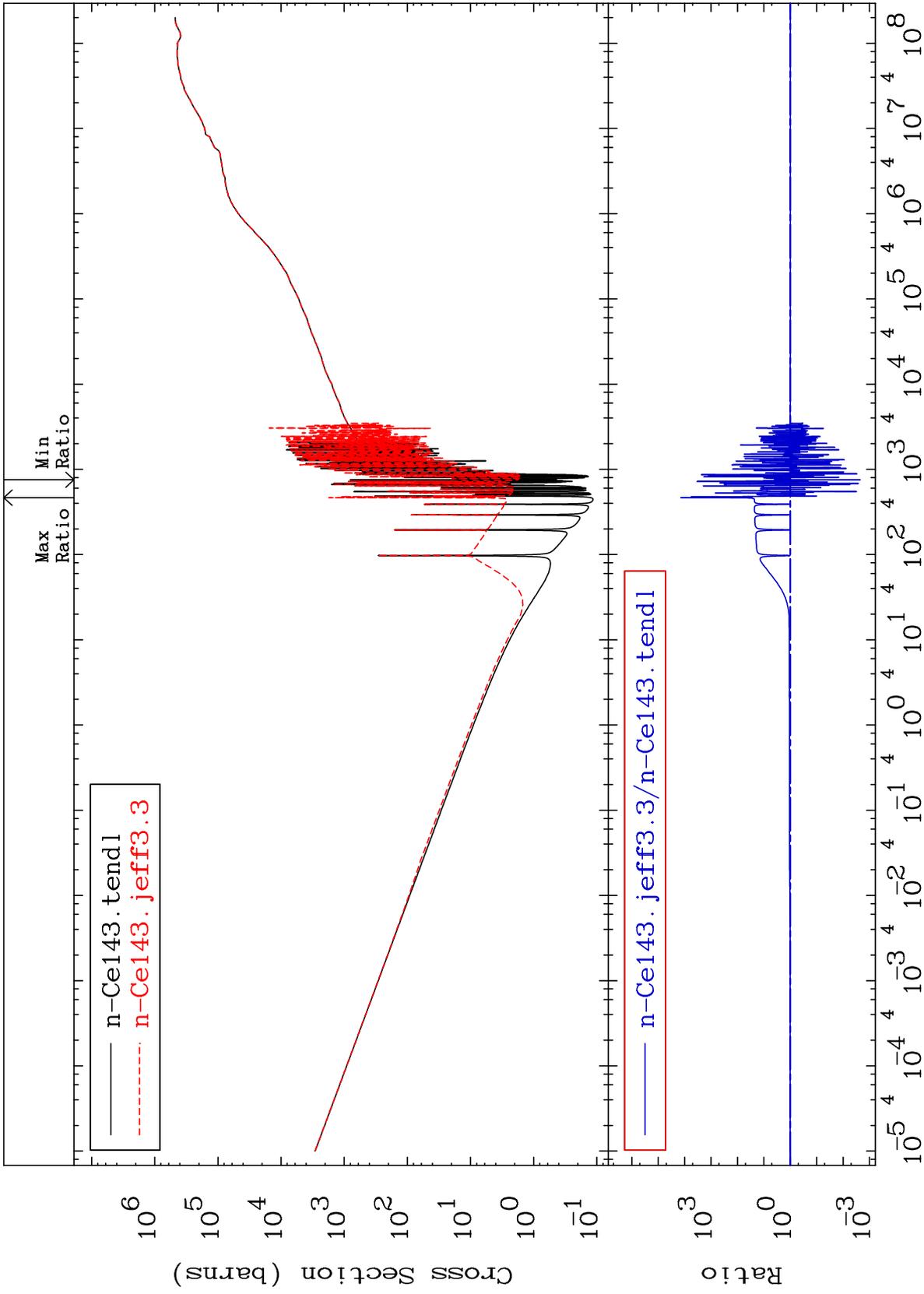
Incident Energy (eV)

58-Ce-143





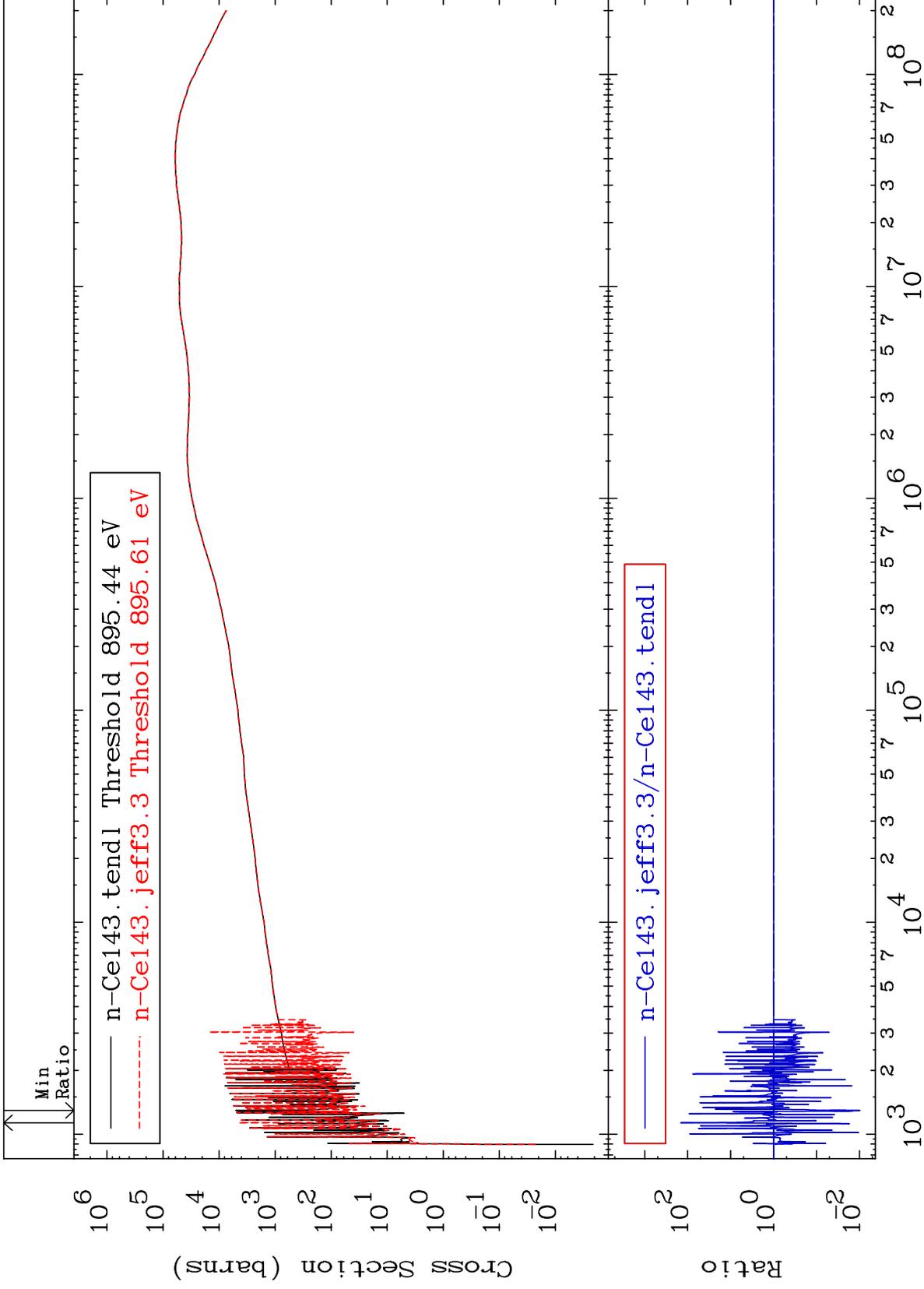




MAT 5846

Dpa elastic (mt2)
Cross Section

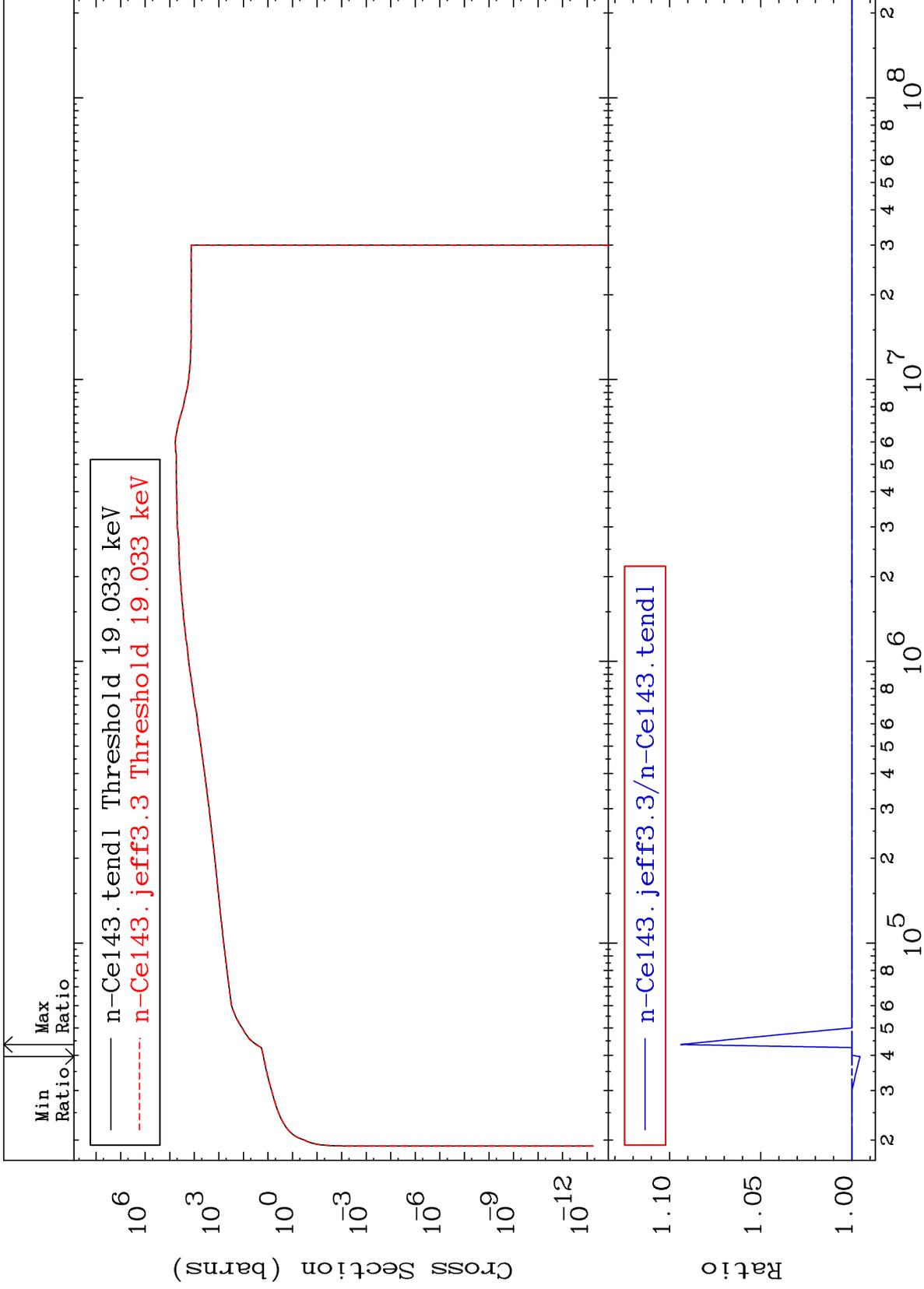
58-Ce-143
-99.03 To 9999. %

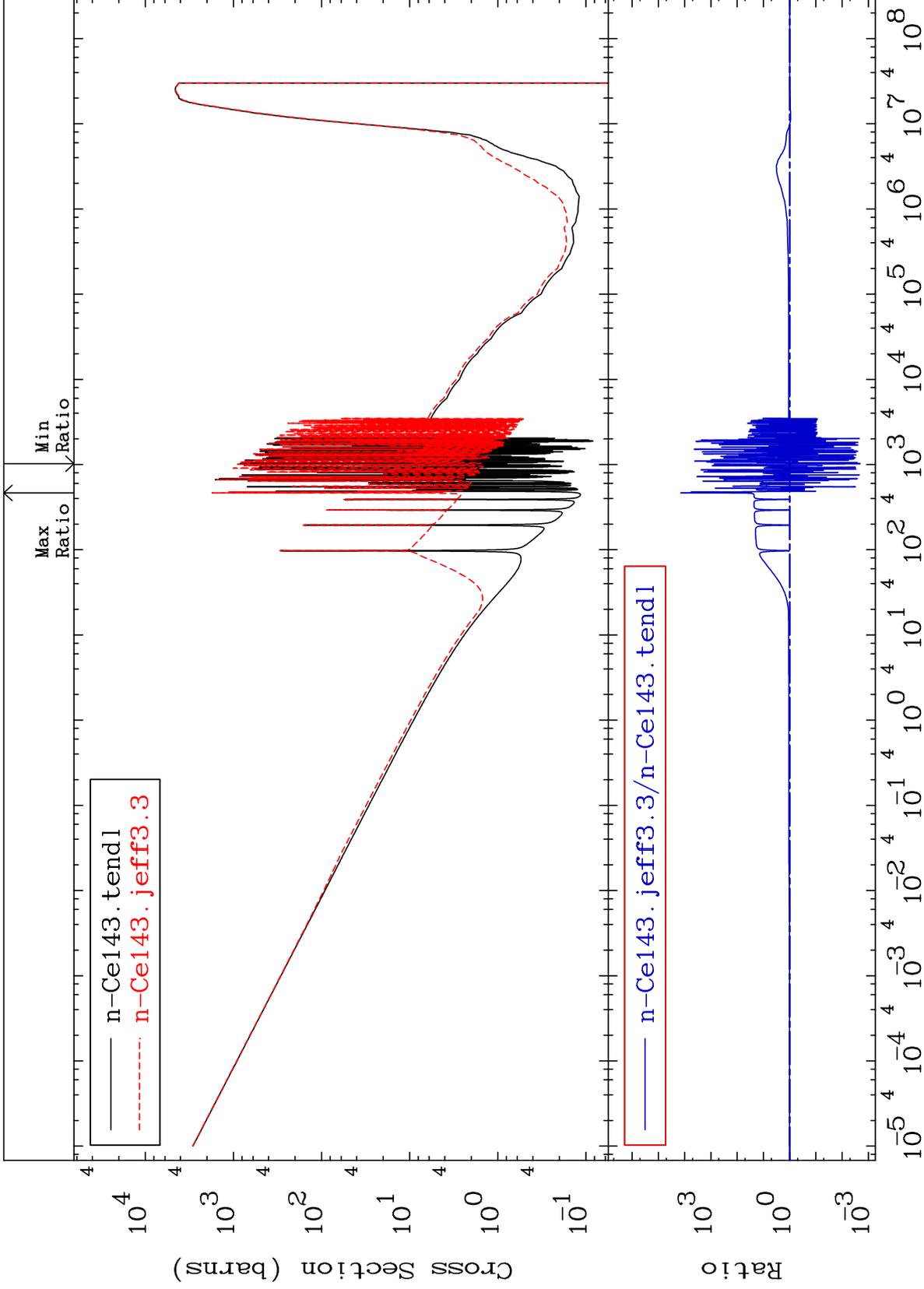


45

Incident Energy (eV)

58-Ce-143



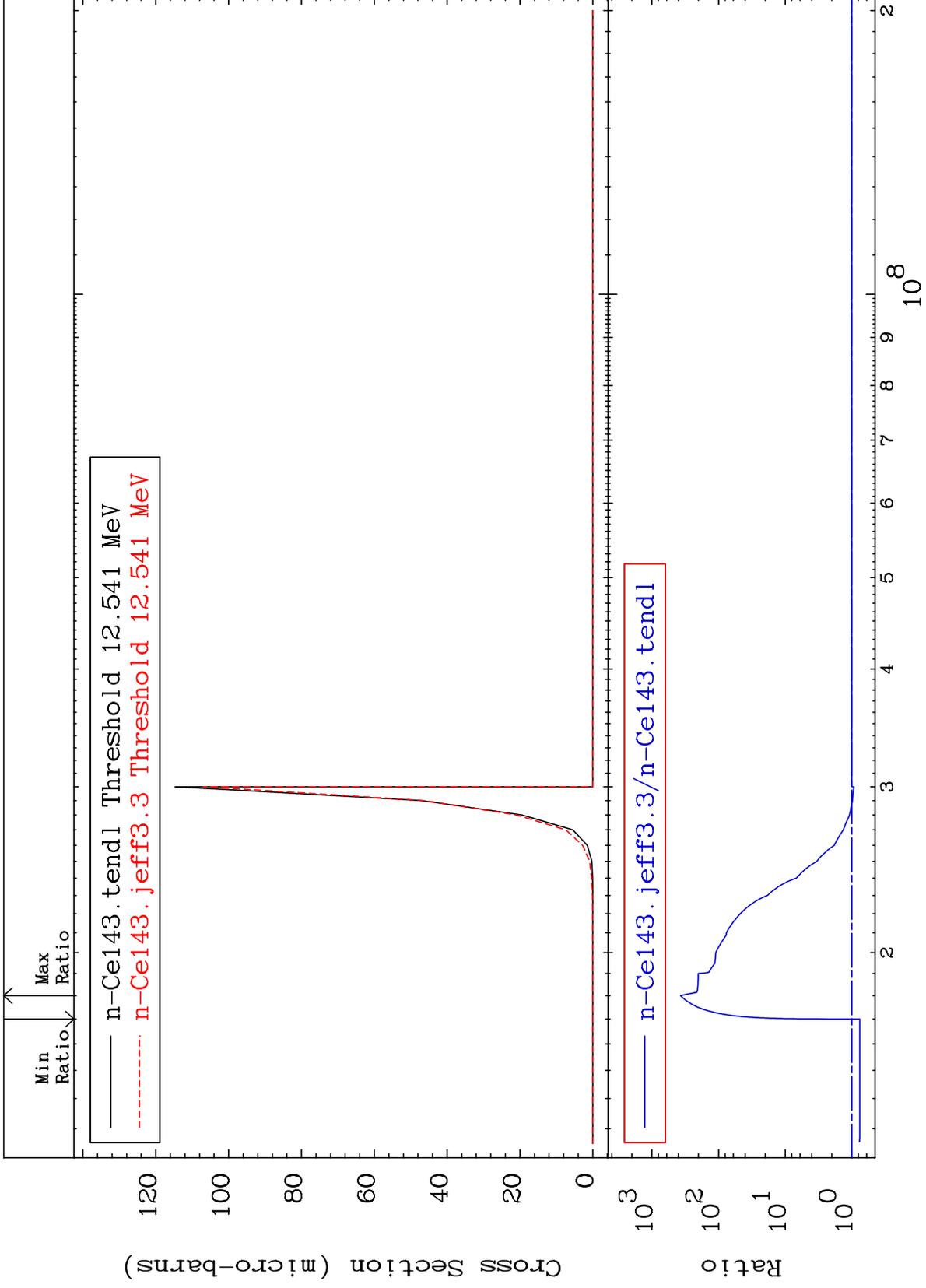


MAT 5846

(n,3n) α :56-Ba-137g

58-Ce-143

Radionuclide Production Cross Section -23.73 To 9999. %

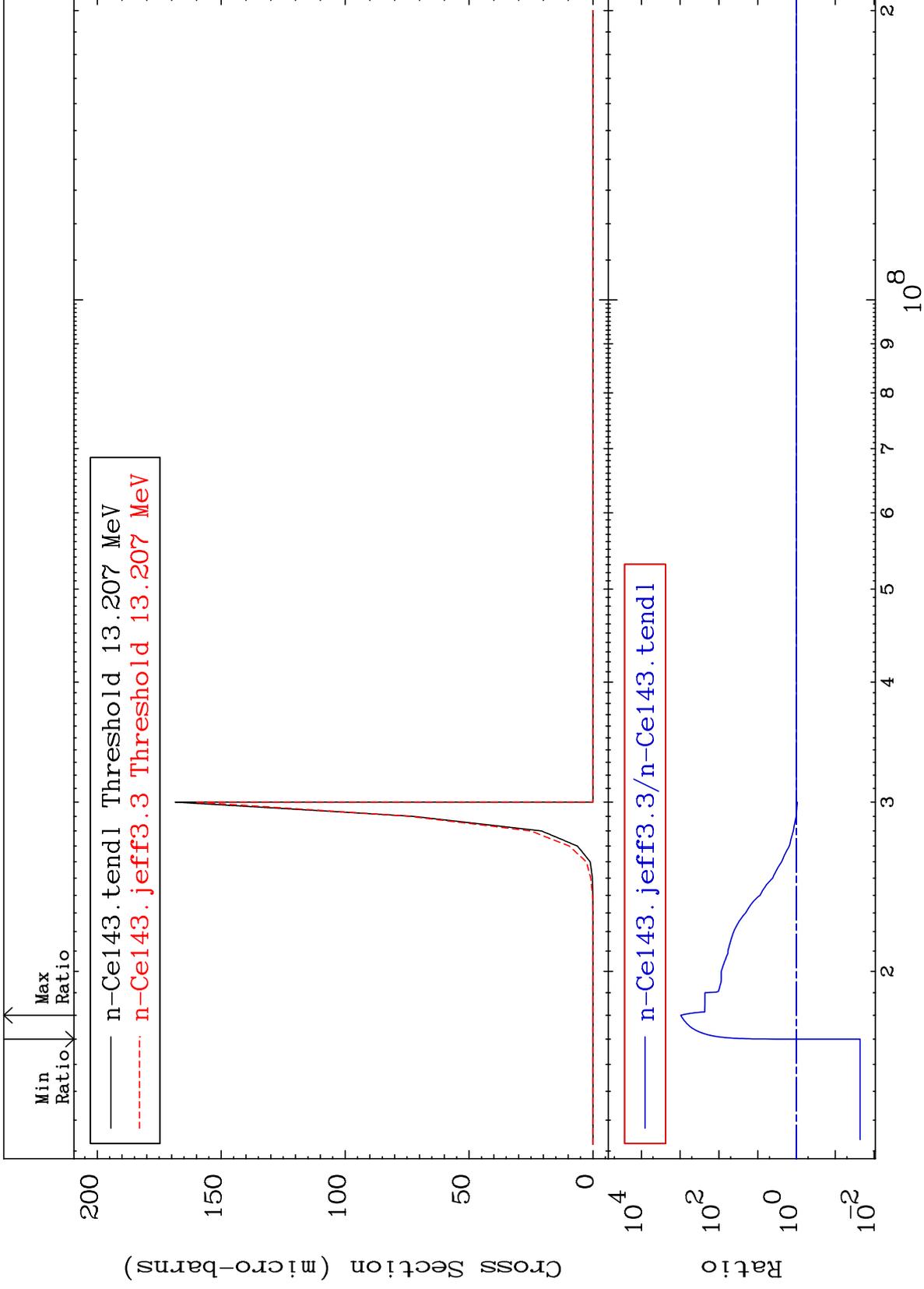


MAT 5846

(n,3n) α :56-Ba-137m2

58-Ce-143

Radionuclide Production Cross Section -97.71 To 9999. %



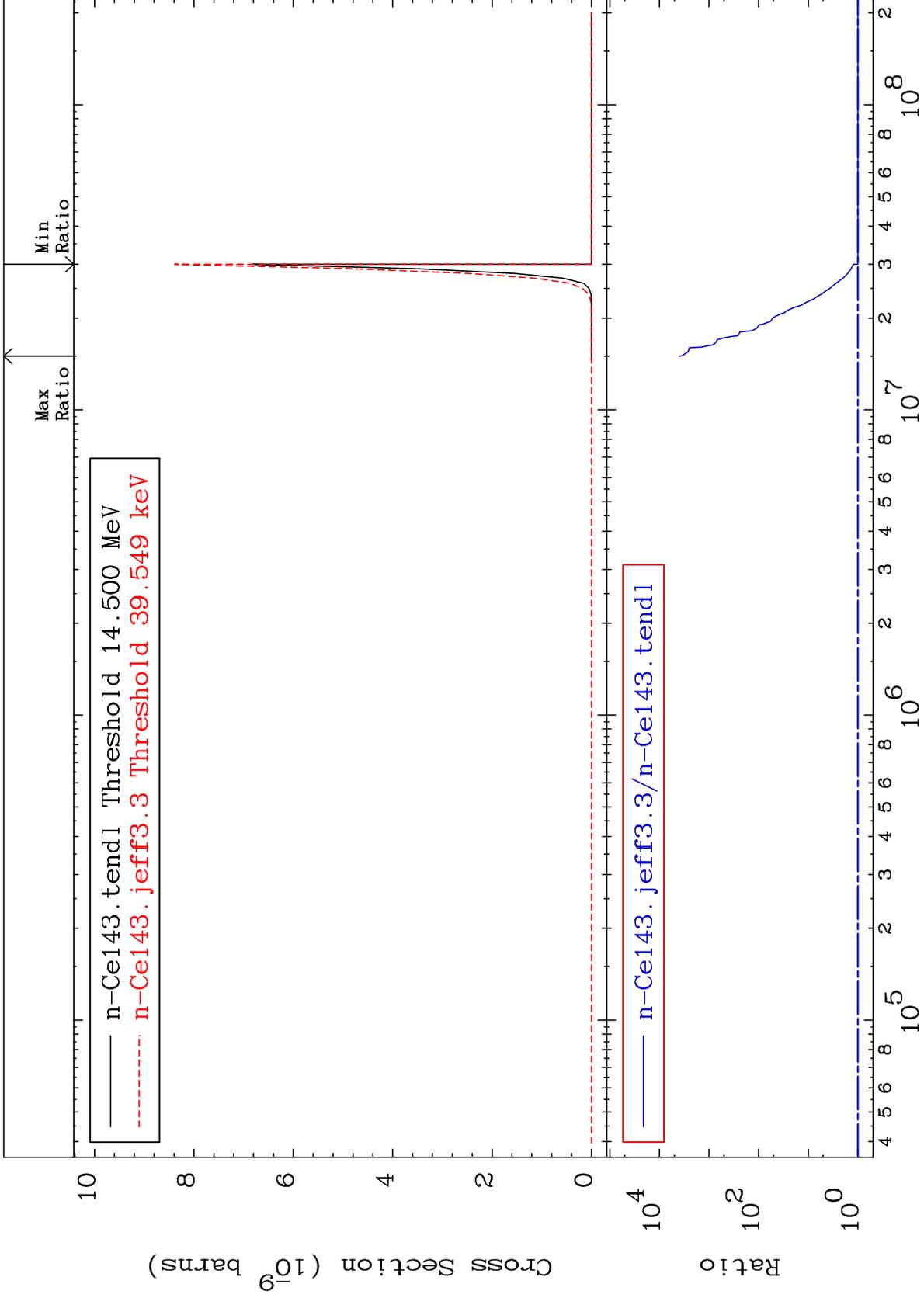
MAT 5846

(n,n') 2α:54-Xe-135g

58-Ce-143

Radionuclide Production Cross Section

0.000 To 9999. %

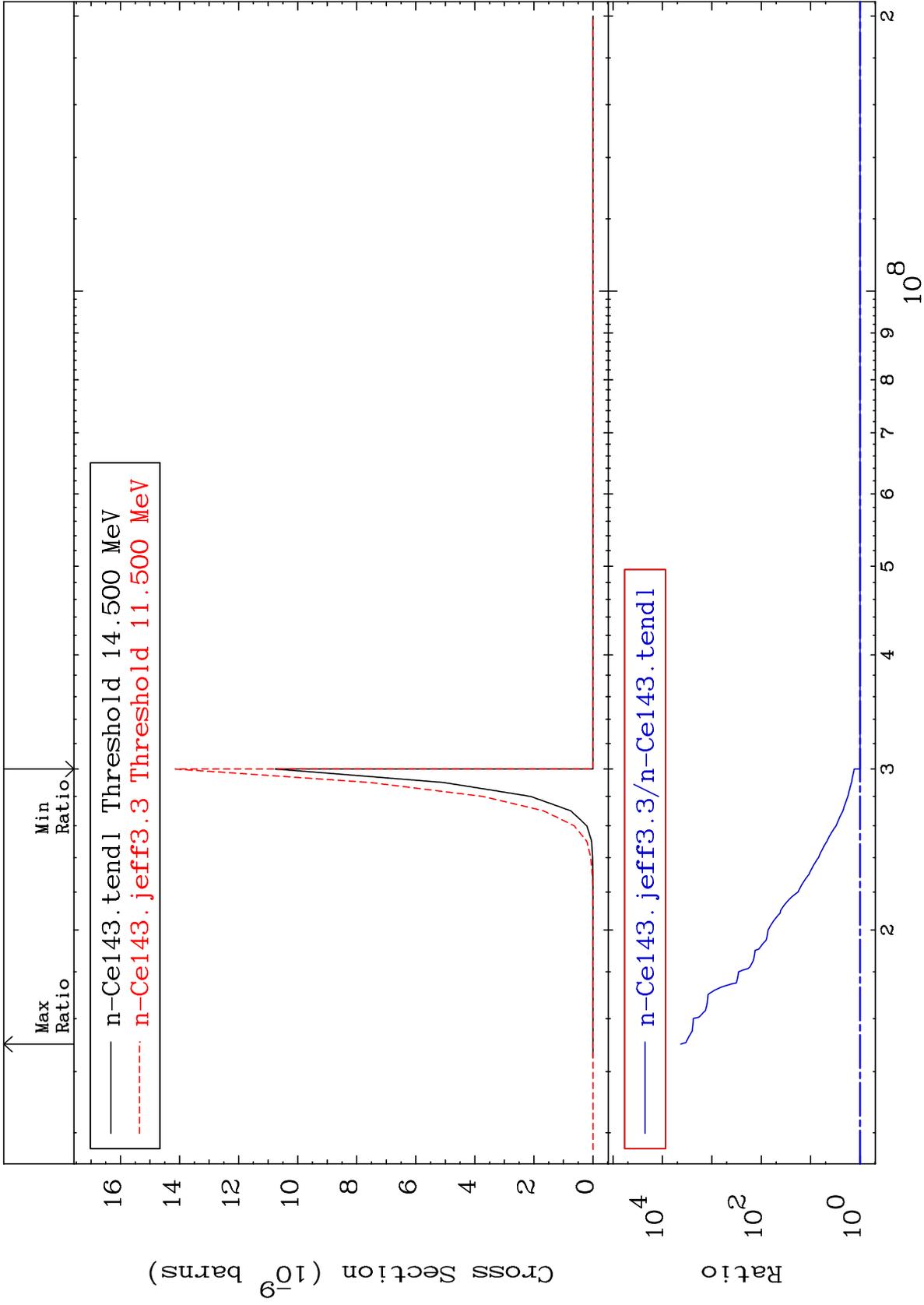


MAT 5846

(n, n') $2\alpha:54\text{-Xe-135m2}$

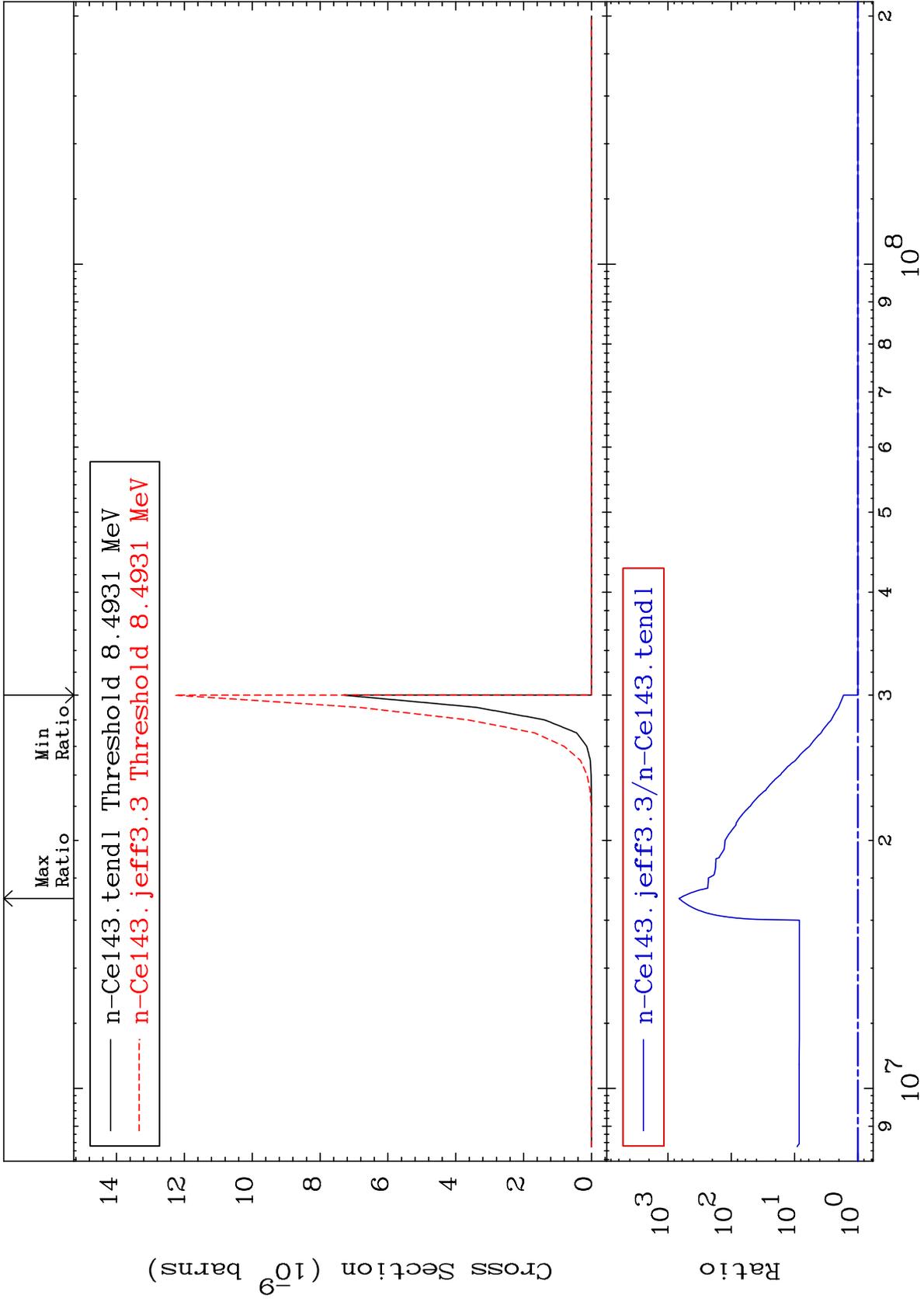
58-Ce-143

Radionuclide Production Cross Section 0.000 To 9999. %



MAT 5846

(n, n') p α :55-Cs-138g 58-Ce-143
Radionuclide Production Cross Section 0.000 To 9999. %



58-Ce-143

Incident Energy (eV)

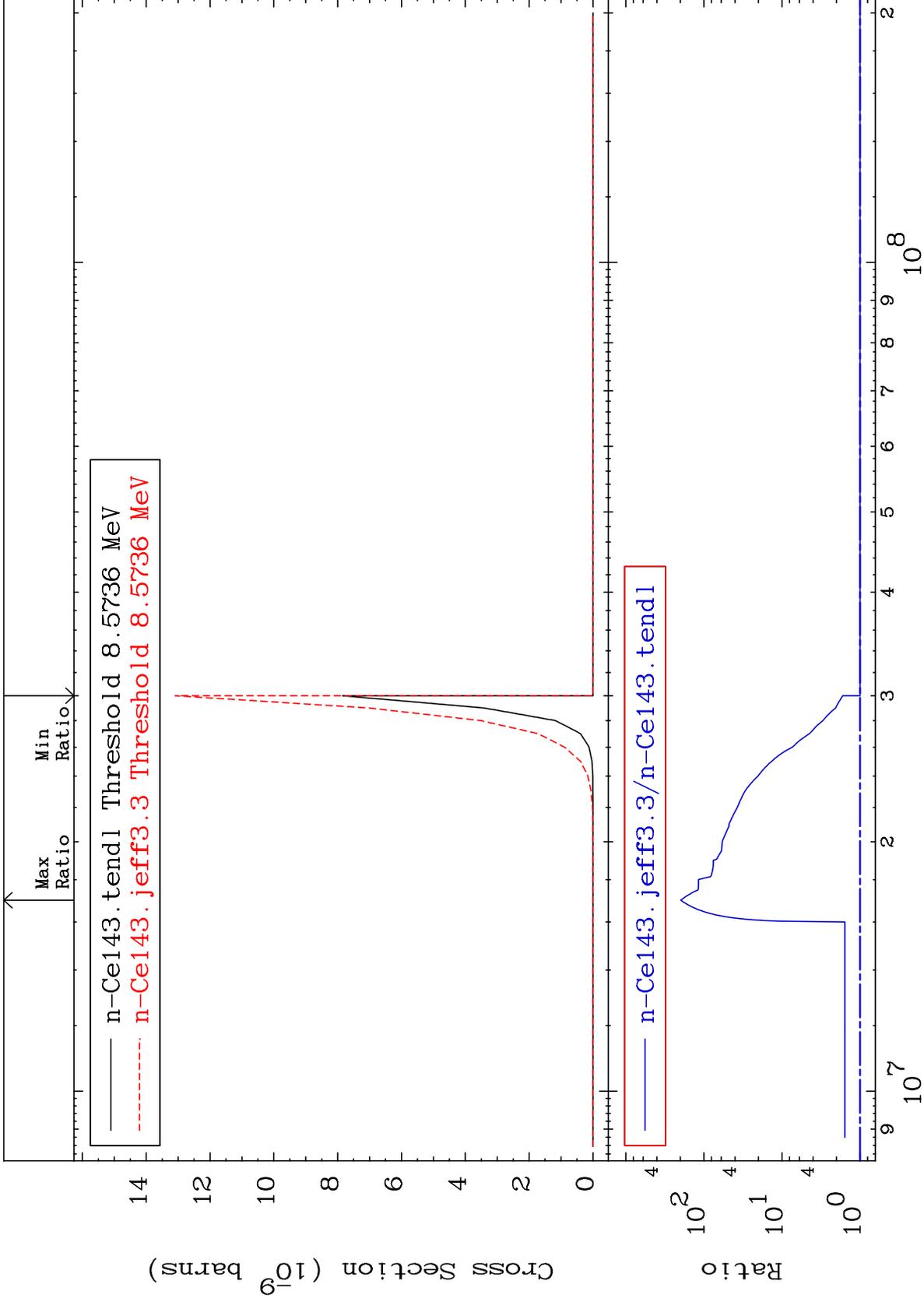
52

MAT 5846

(n, n') p α :55-Cs-138m3

58-Ce-143

Radionuclide Production Cross Section 0.000 To 9999. %



53

Incident Energy (eV)

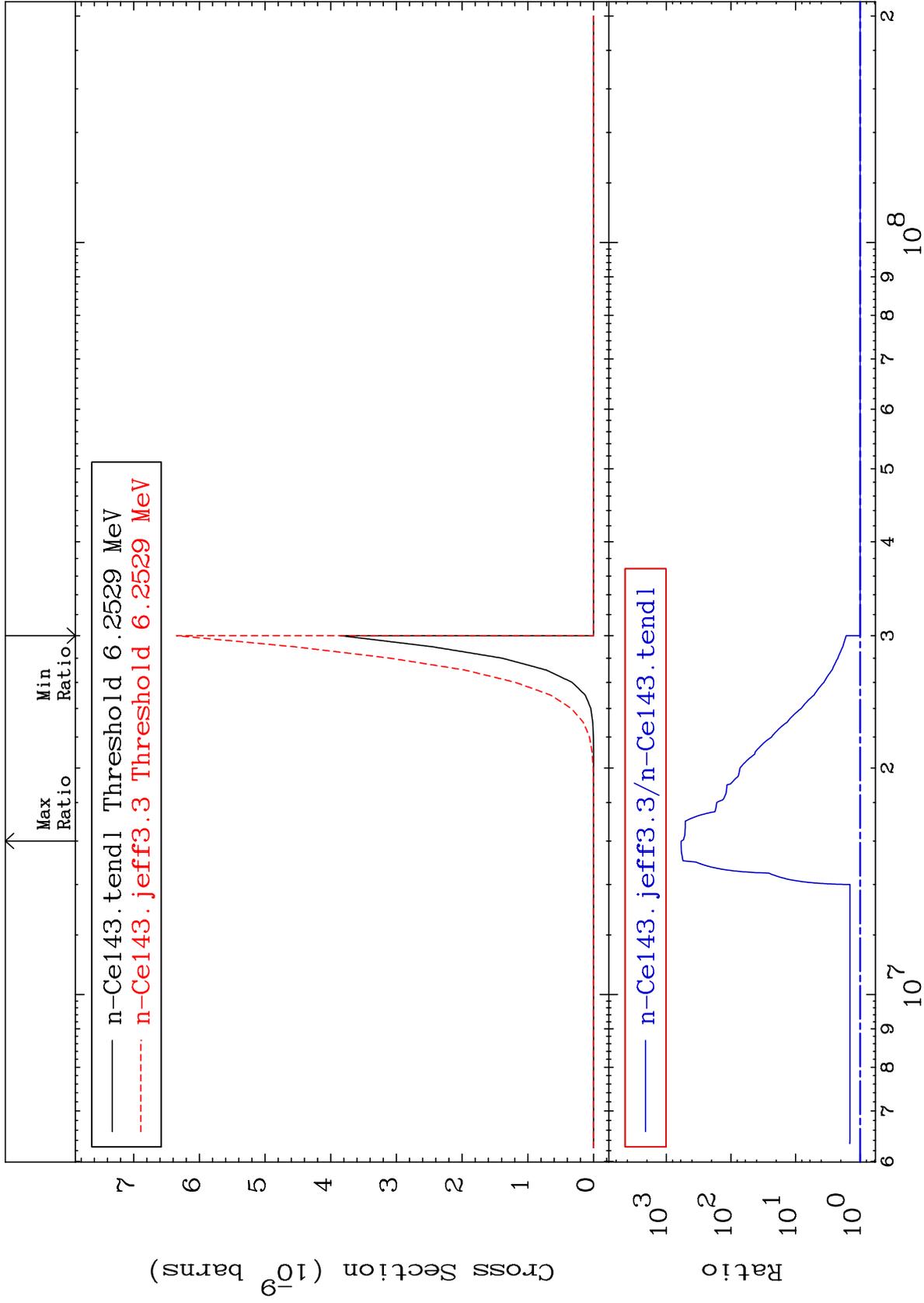
58-Ce-143

MAT 5846

(n, d) α :55-Cs-138g

58-Ce-143

Radionuclide Production Cross Section 0.000 To 9999. %



54

Incident Energy (eV)

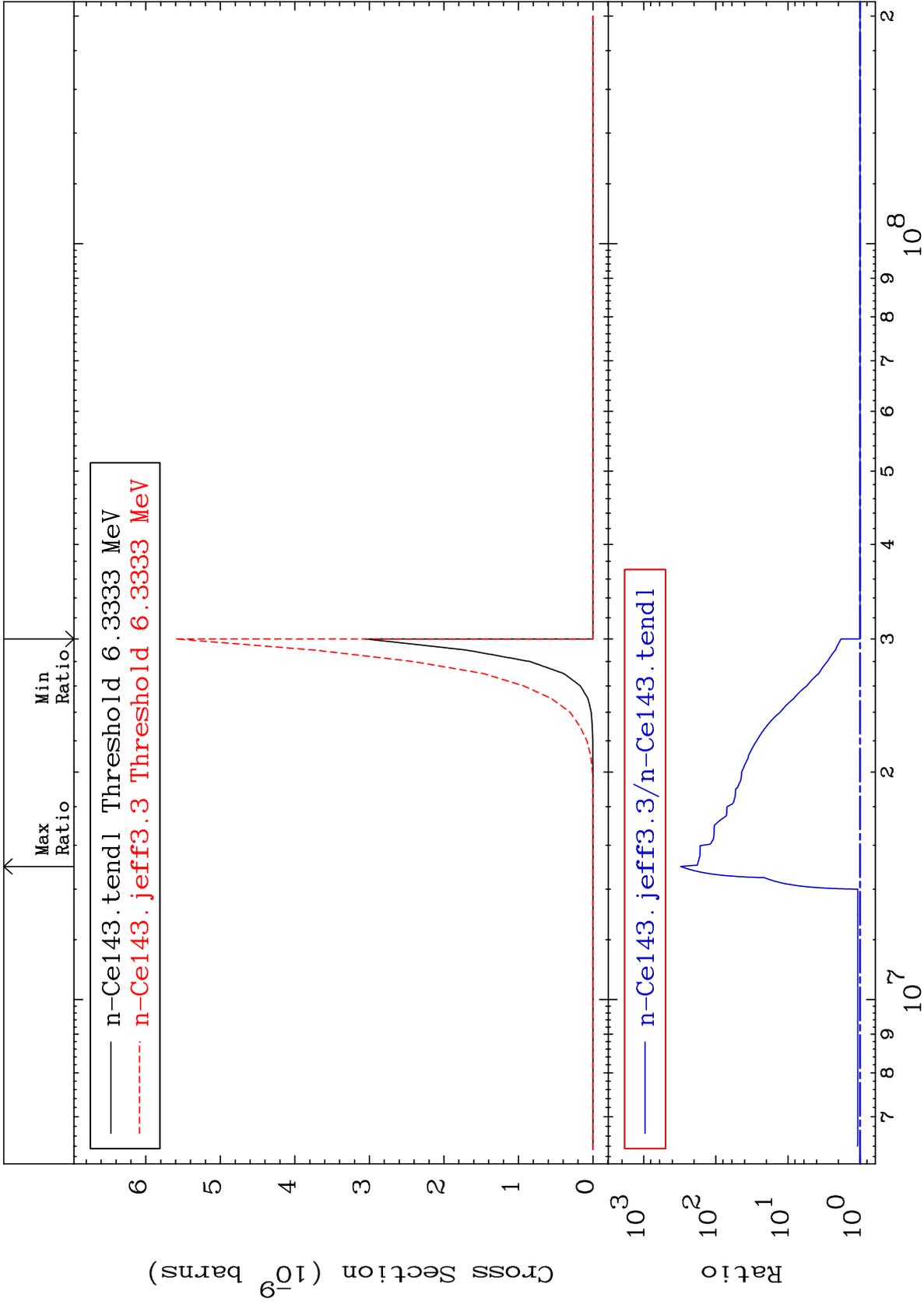
58-Ce-143

MAT 5846

(n, d) α :55-Cs-138m3

58-Ce-143

Radionuclide Production Cross Section 0.000 To 9999. %



55

Incident Energy (eV)

58-Ce-143