

Program EVALPLOT  
(Version 2017-1)

by

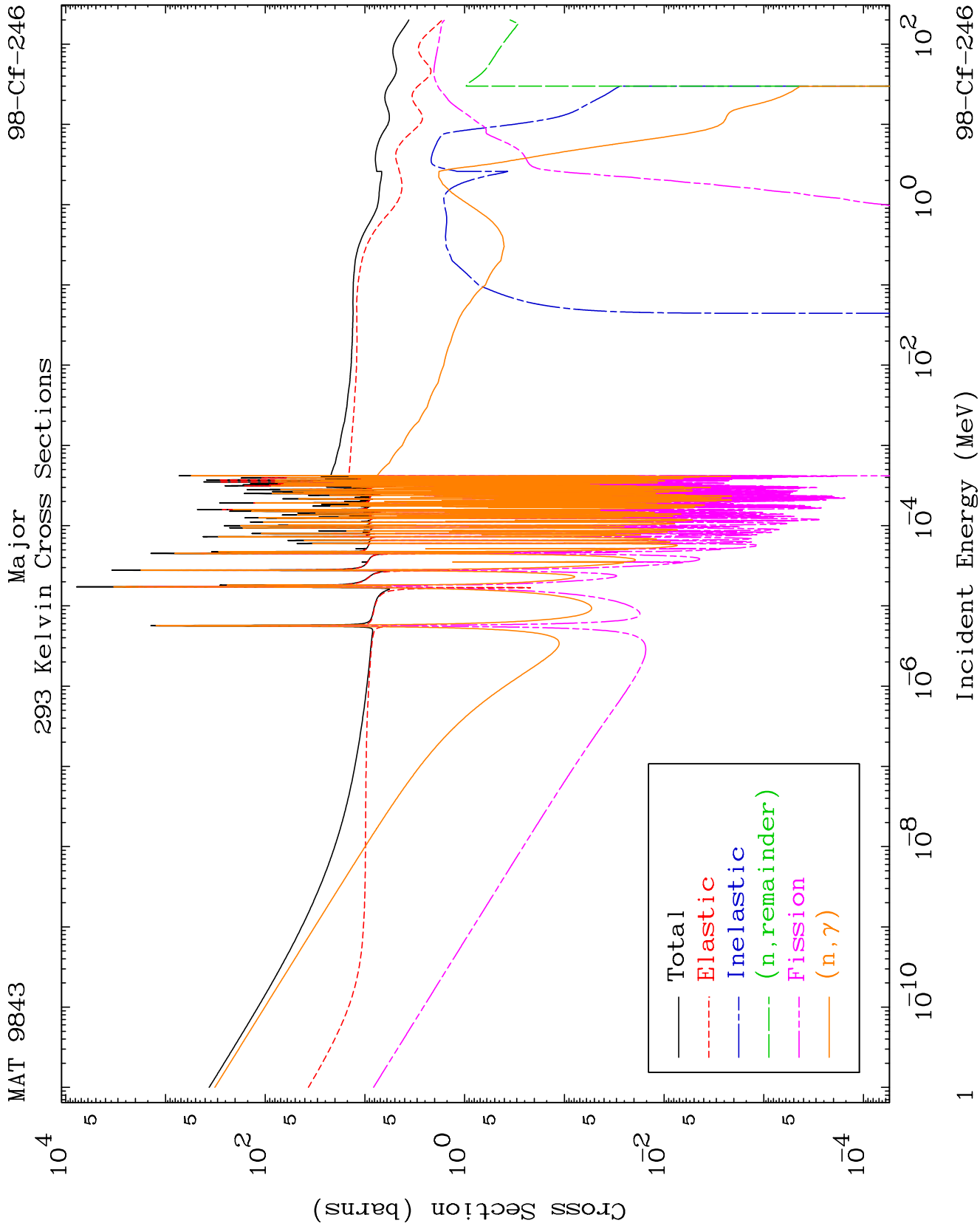
Dermott E. Cullen  
(Present Contact Information)

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Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

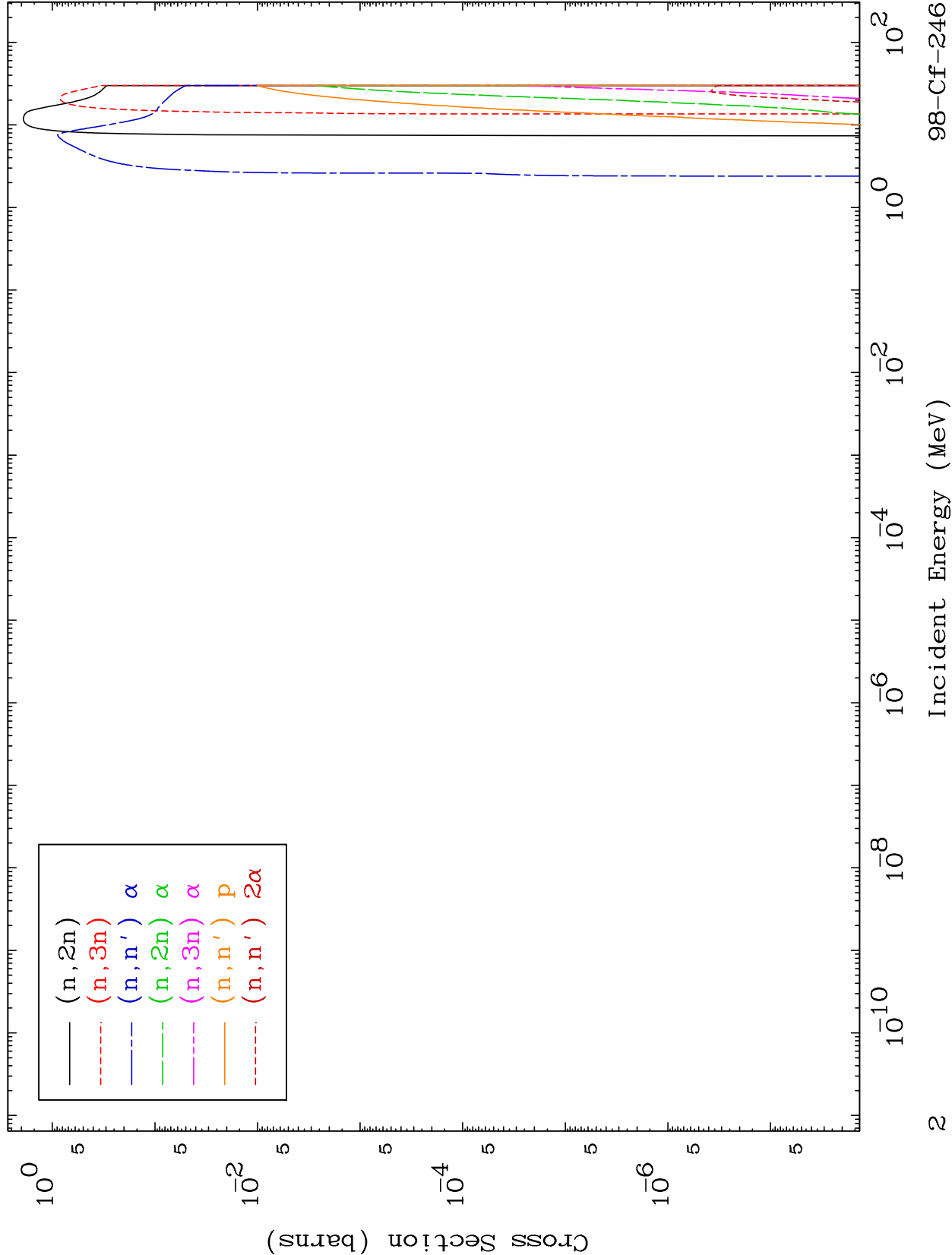
Press Mouse Button to Start



MAT 9843

Neutron Production  
293 Kelvin Cross Sections

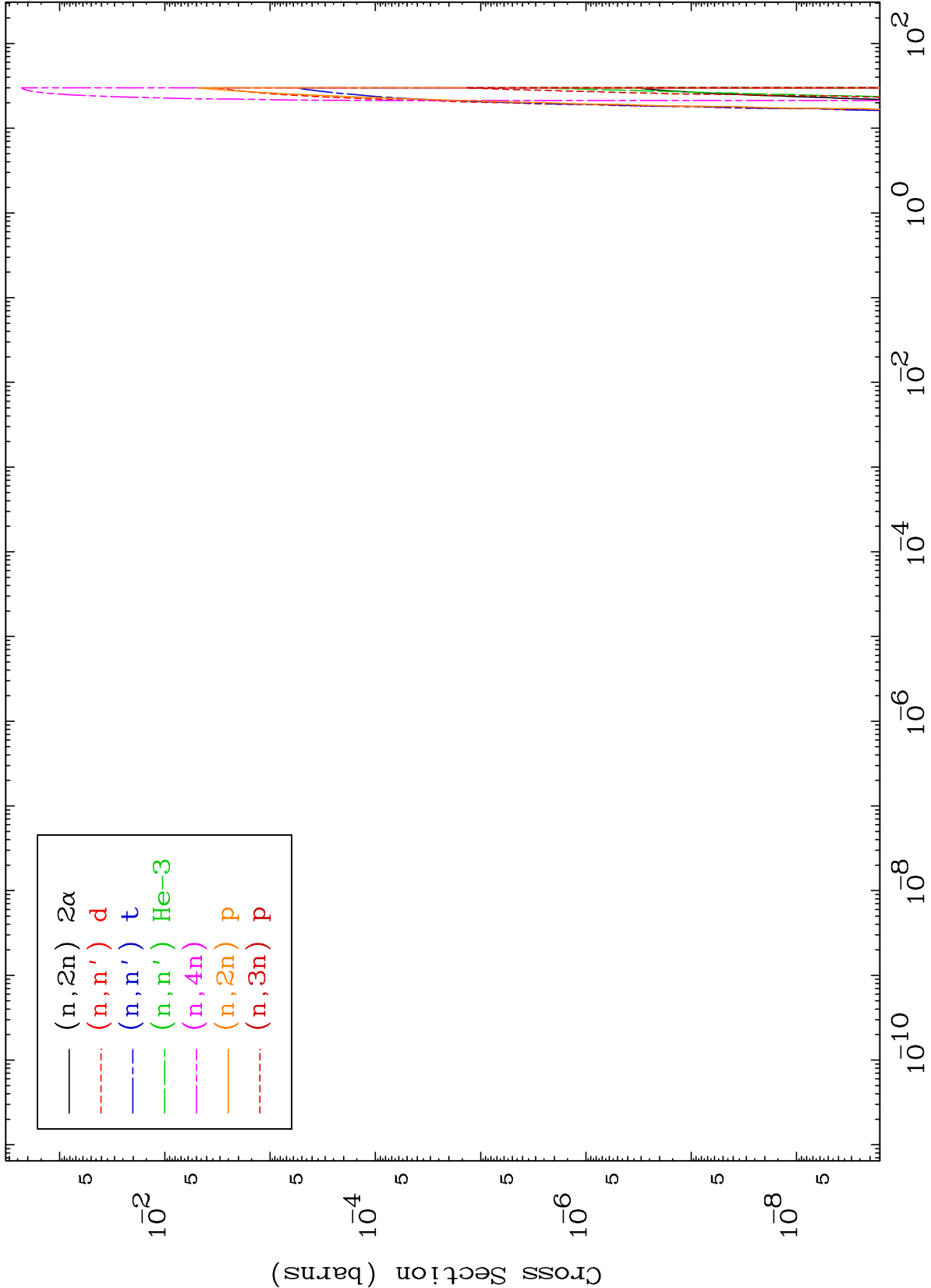
98-Cf-246

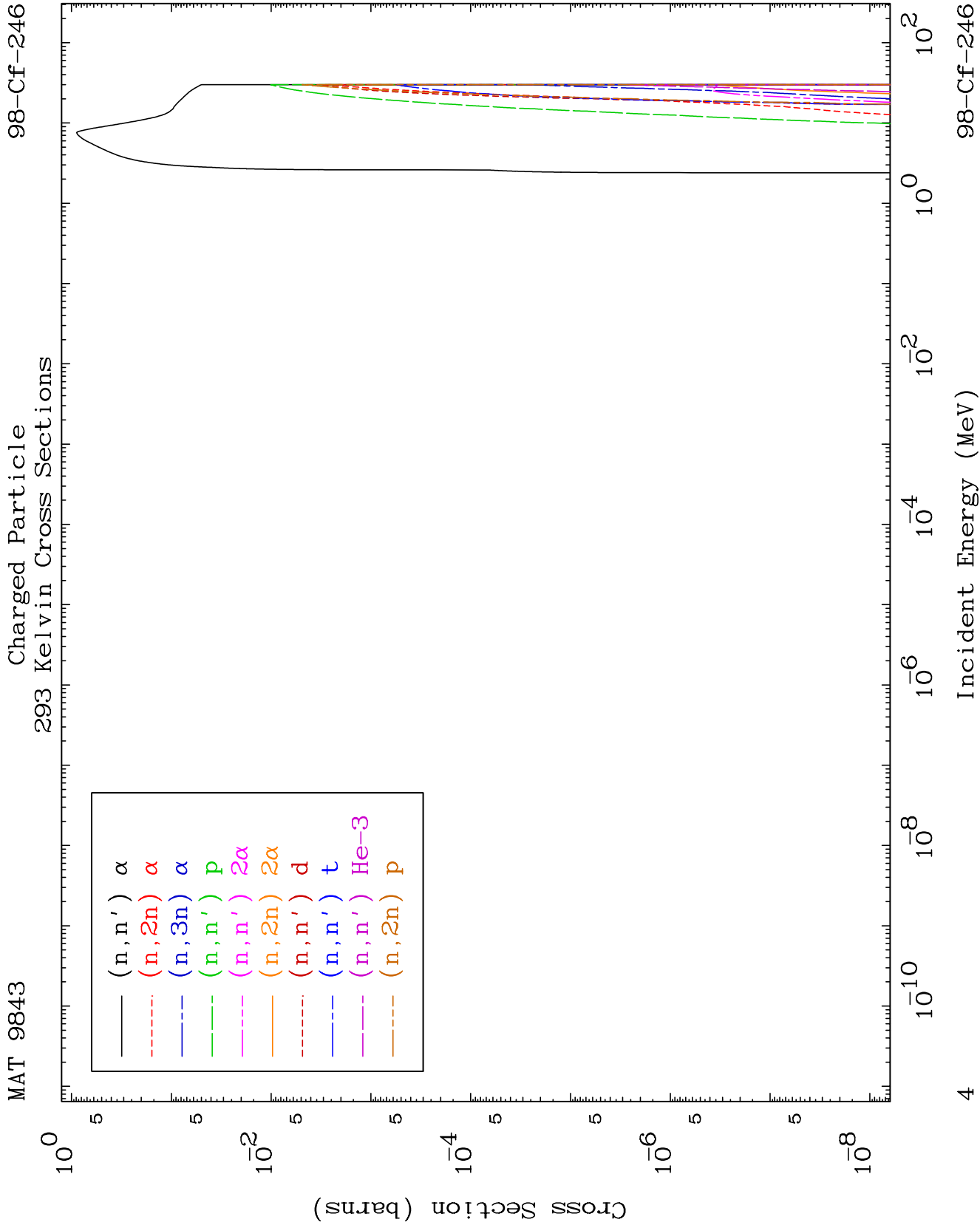


MAT 9843

Neutron Production  
293 Kelvin Cross Sections

98-Cf-246

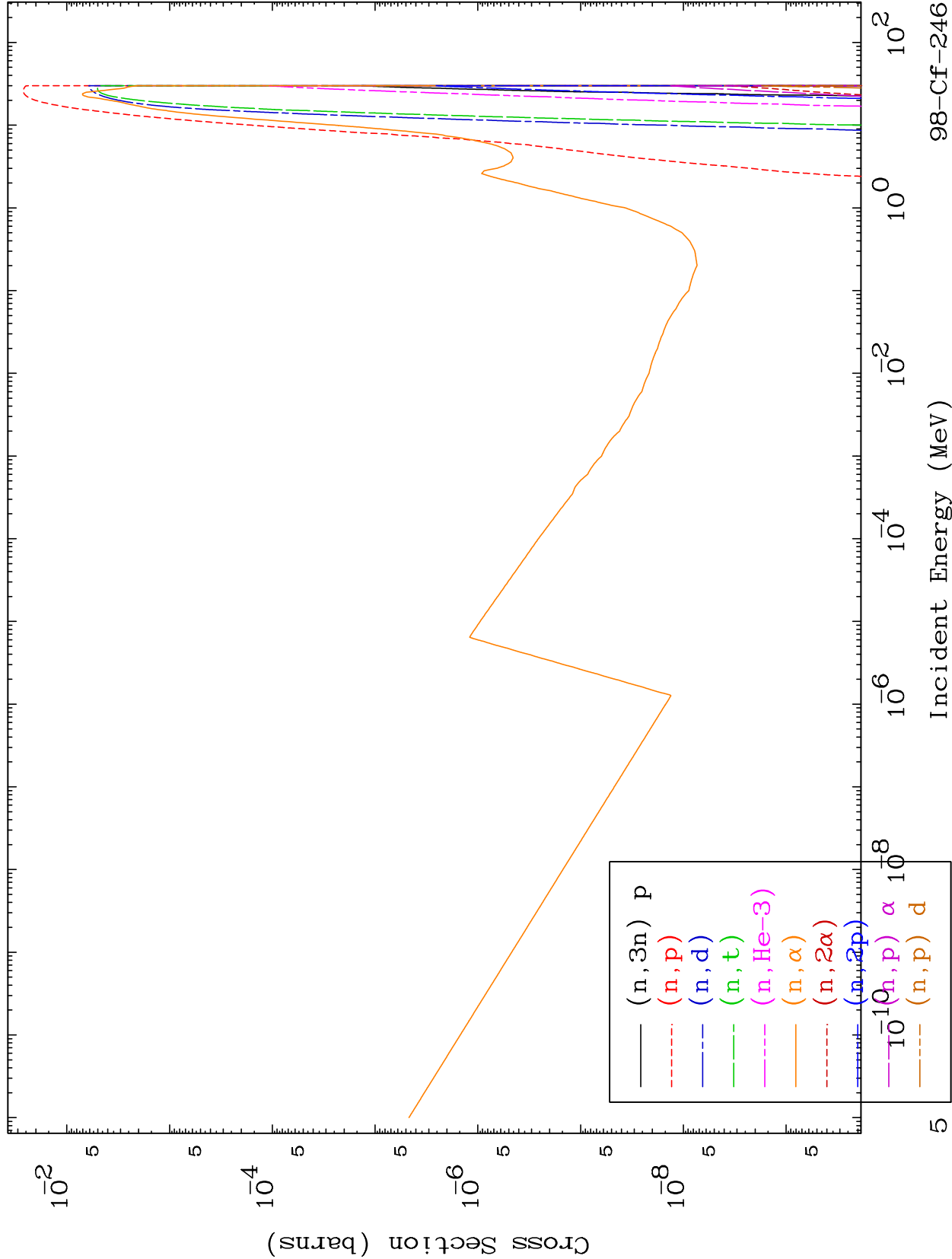


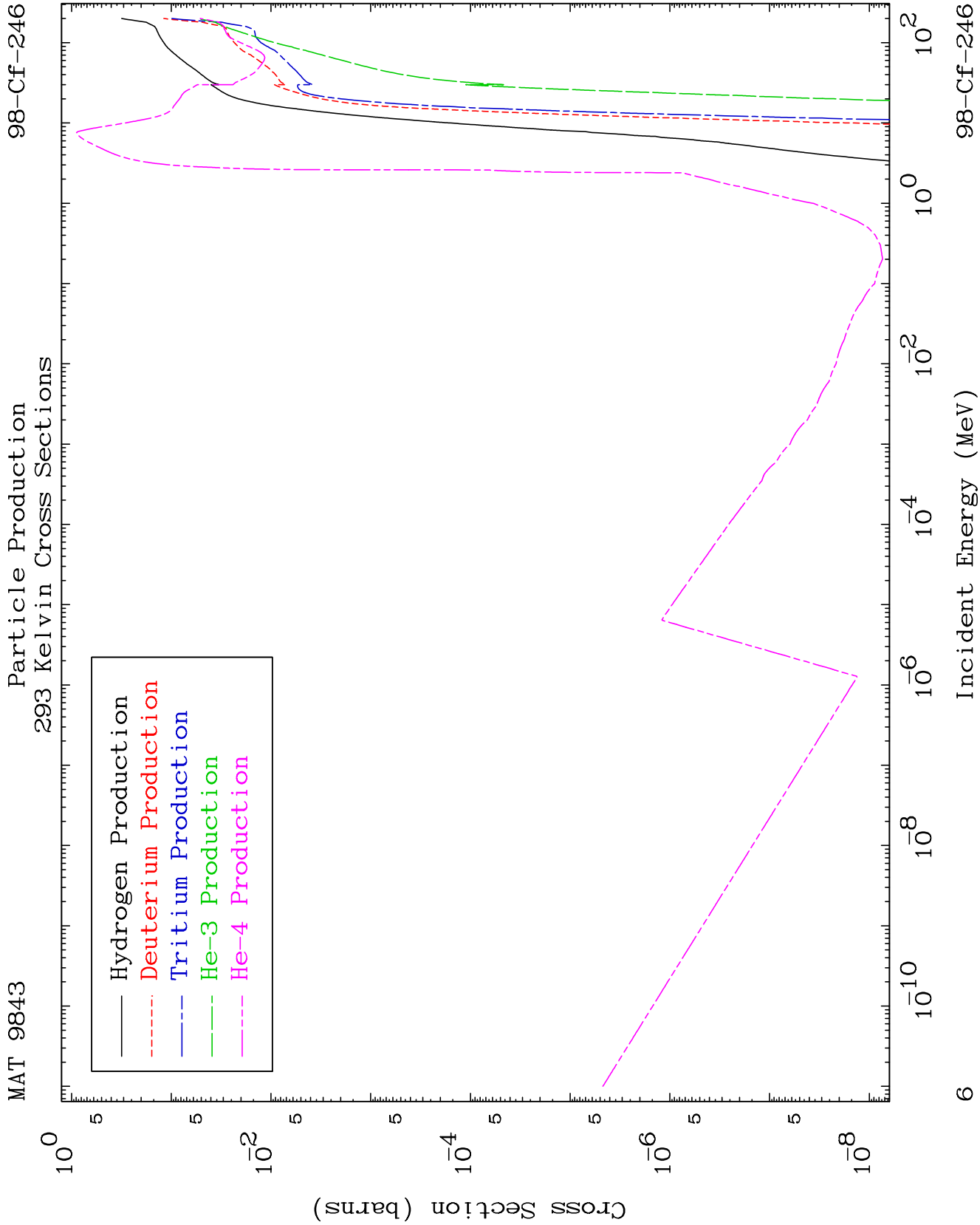


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Charged Particle  
293 Kelvin Cross Sections

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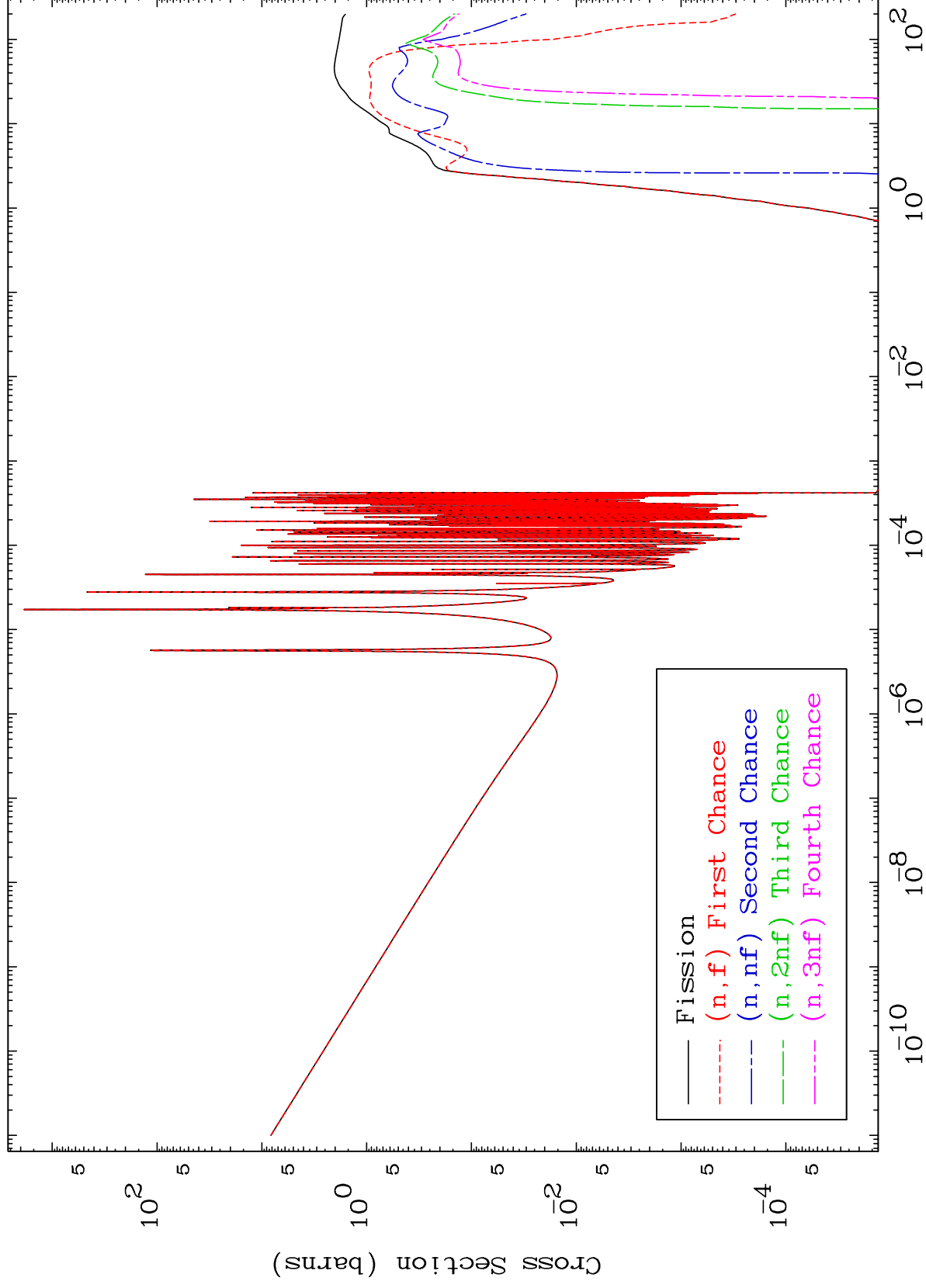


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Fission

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293 Kelvin Cross Sections

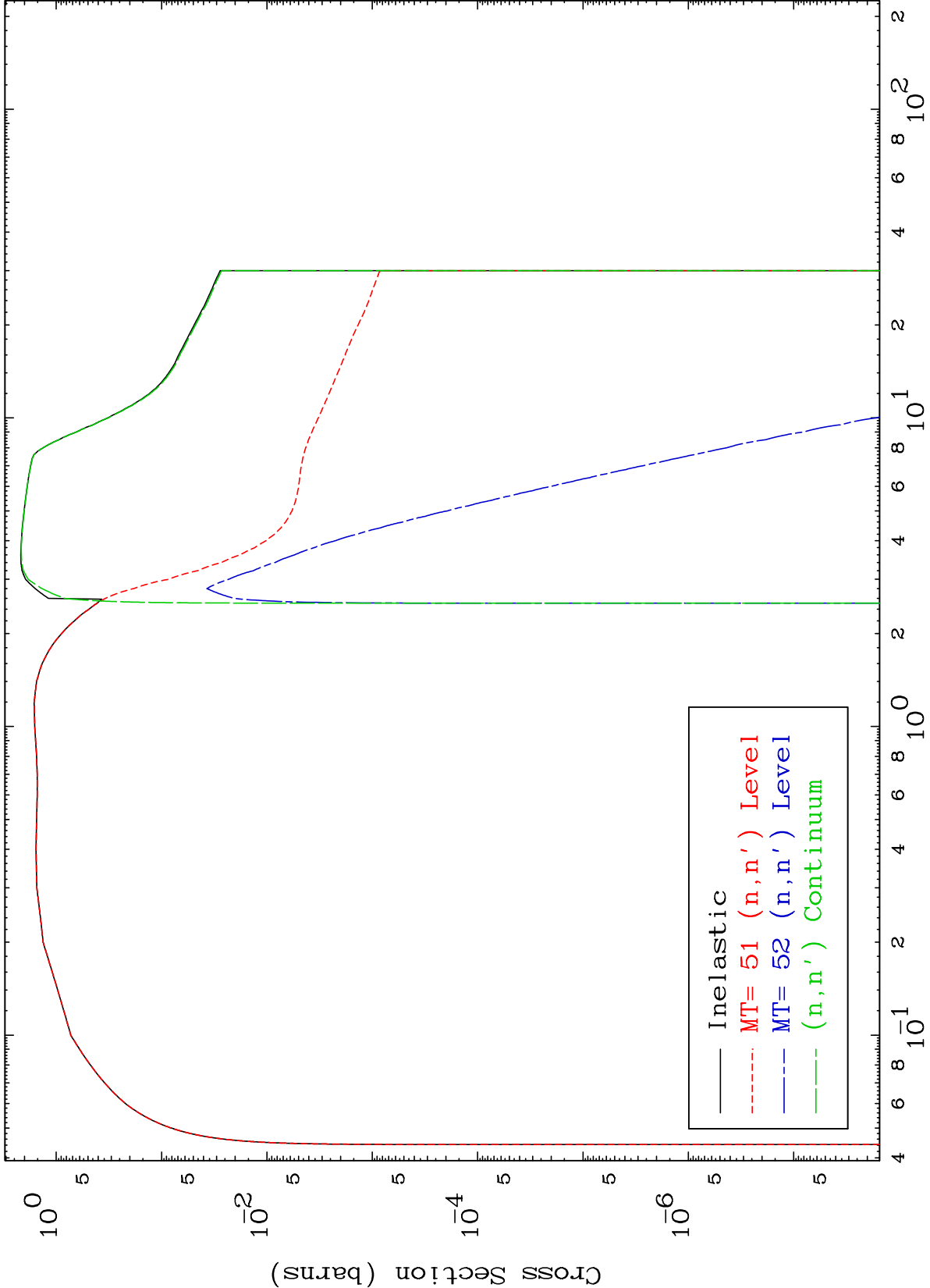


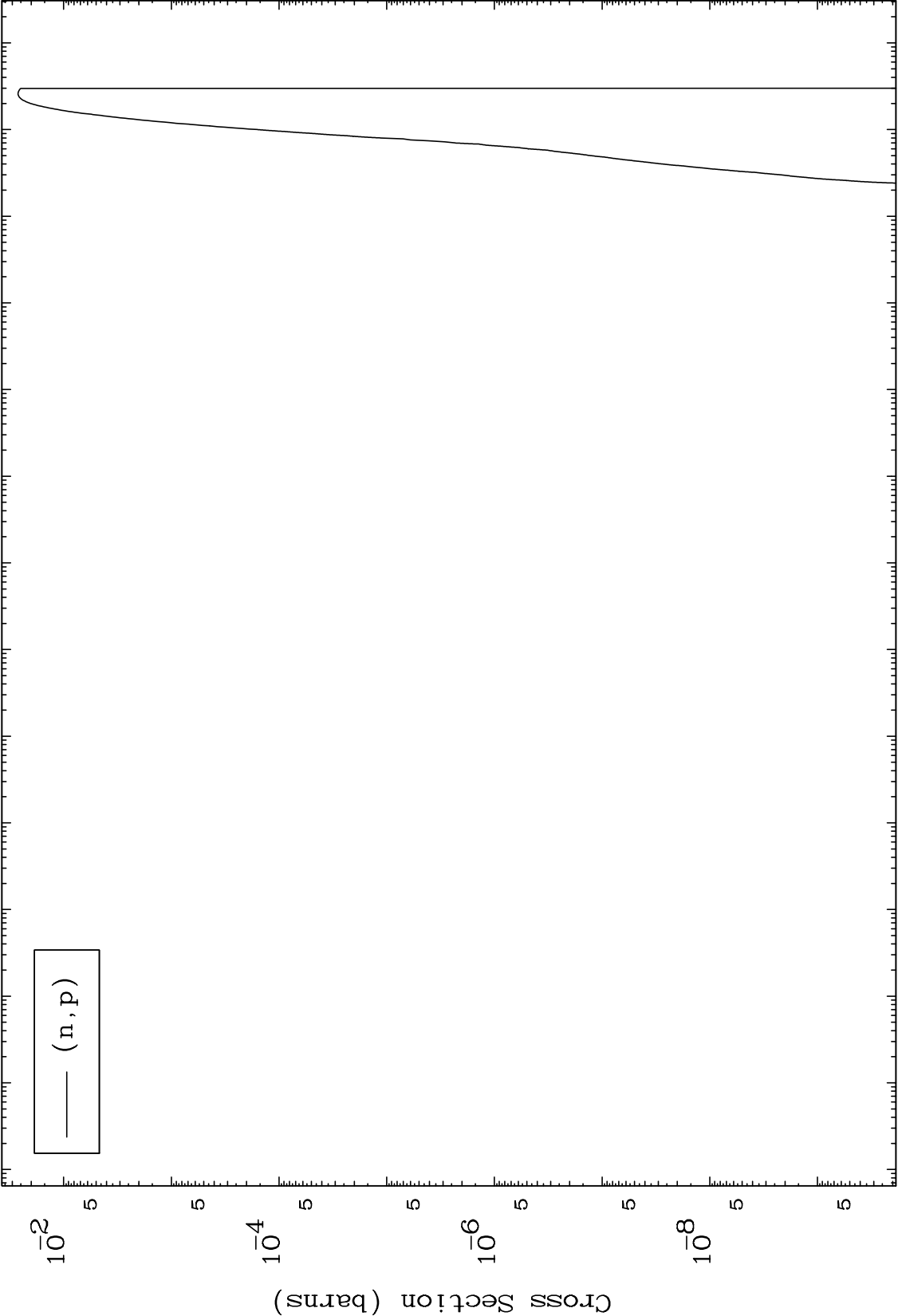


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(n,n') Level  
293 Kelvin Cross Sections

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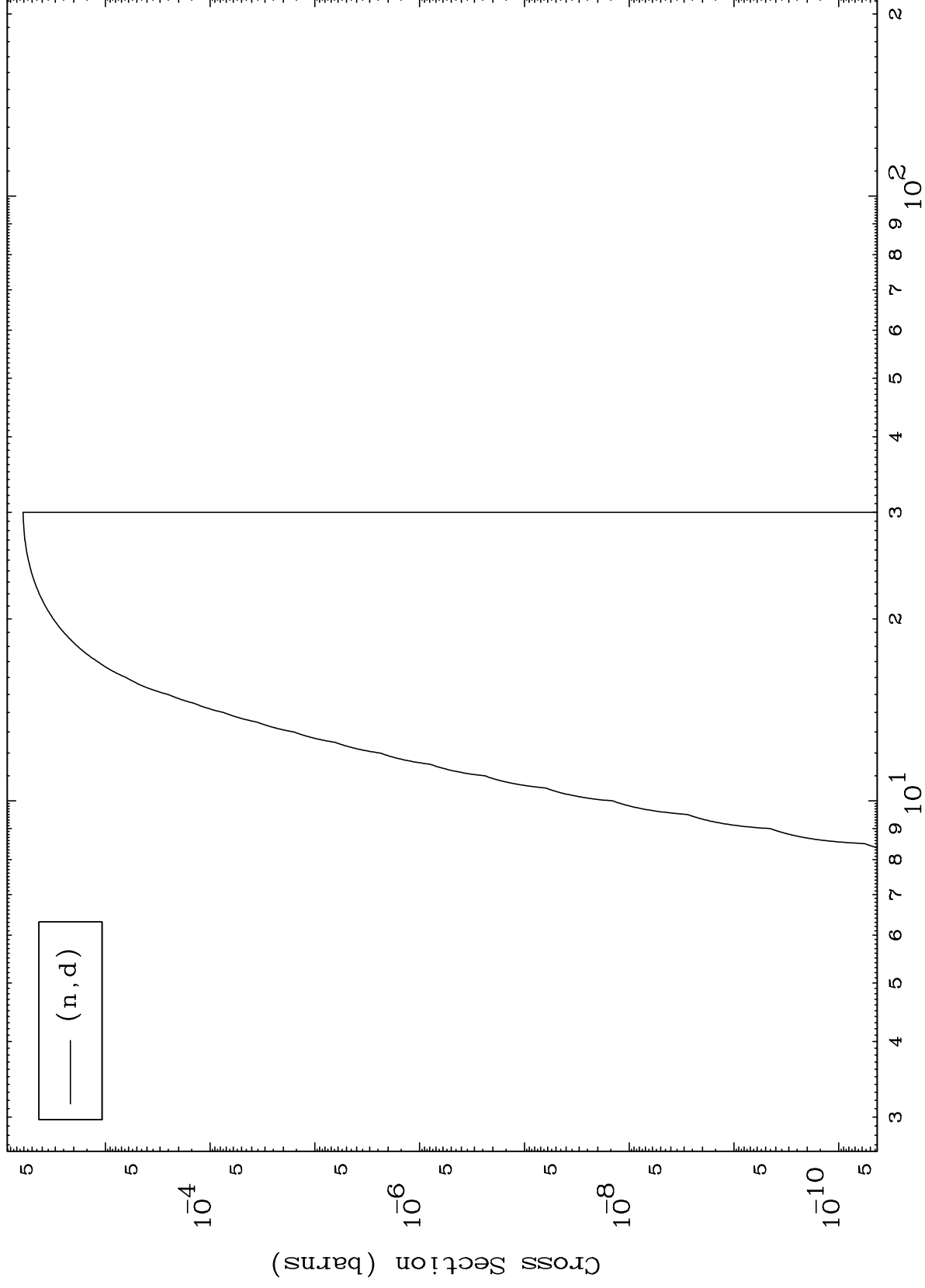




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(n,d) Levels  
293 Kelvin Cross Sections



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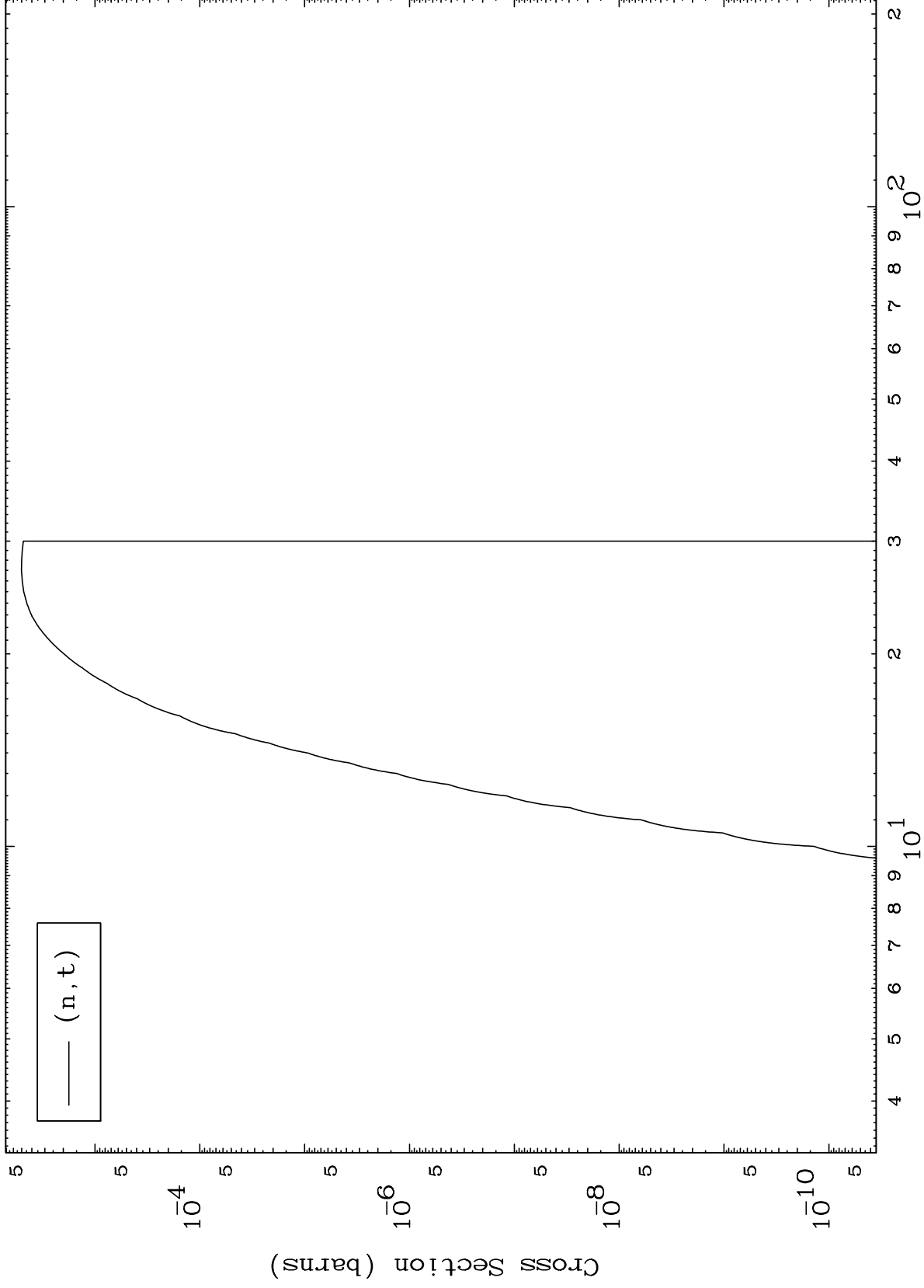
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

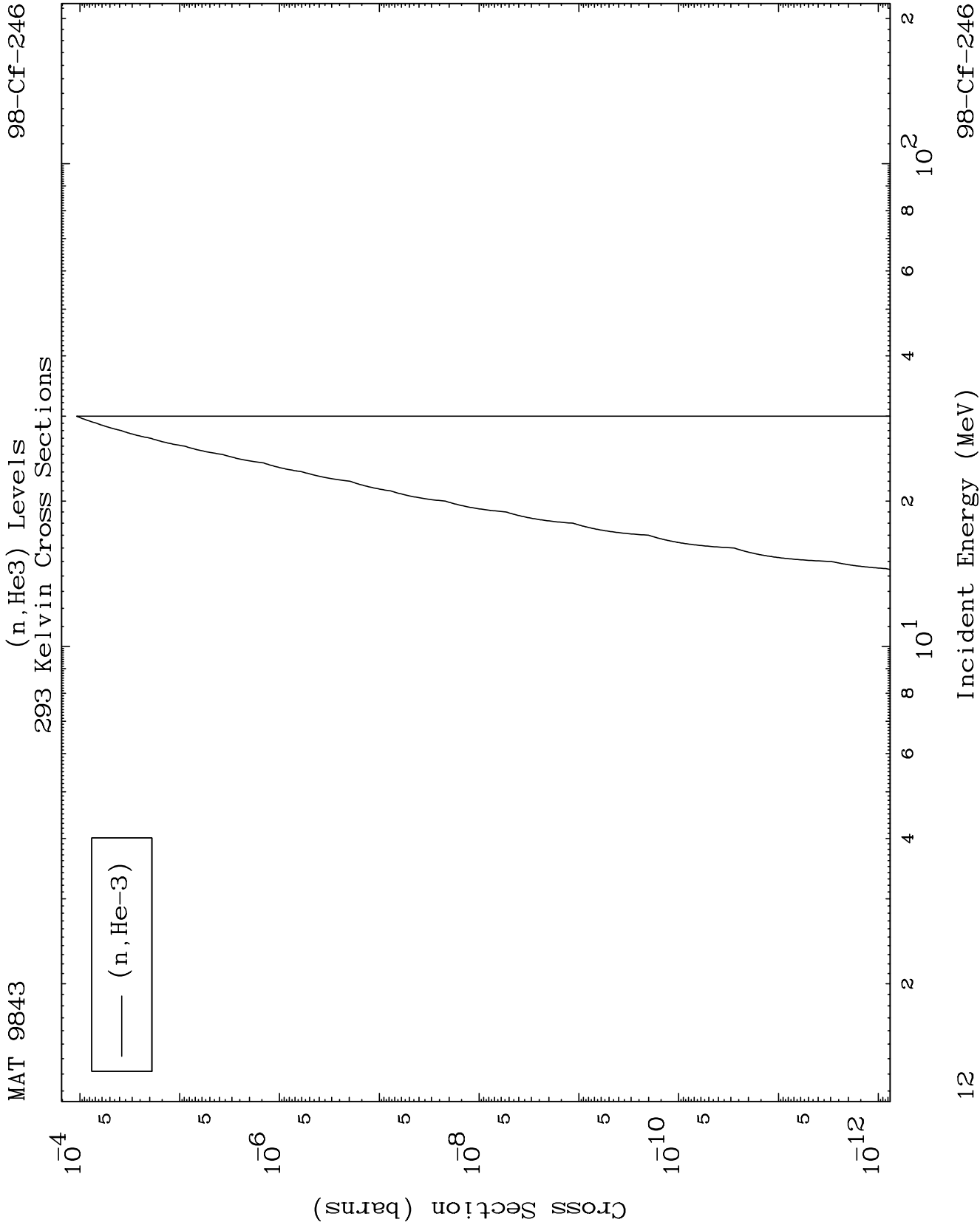
98-Cf-246



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Incident Energy (MeV)

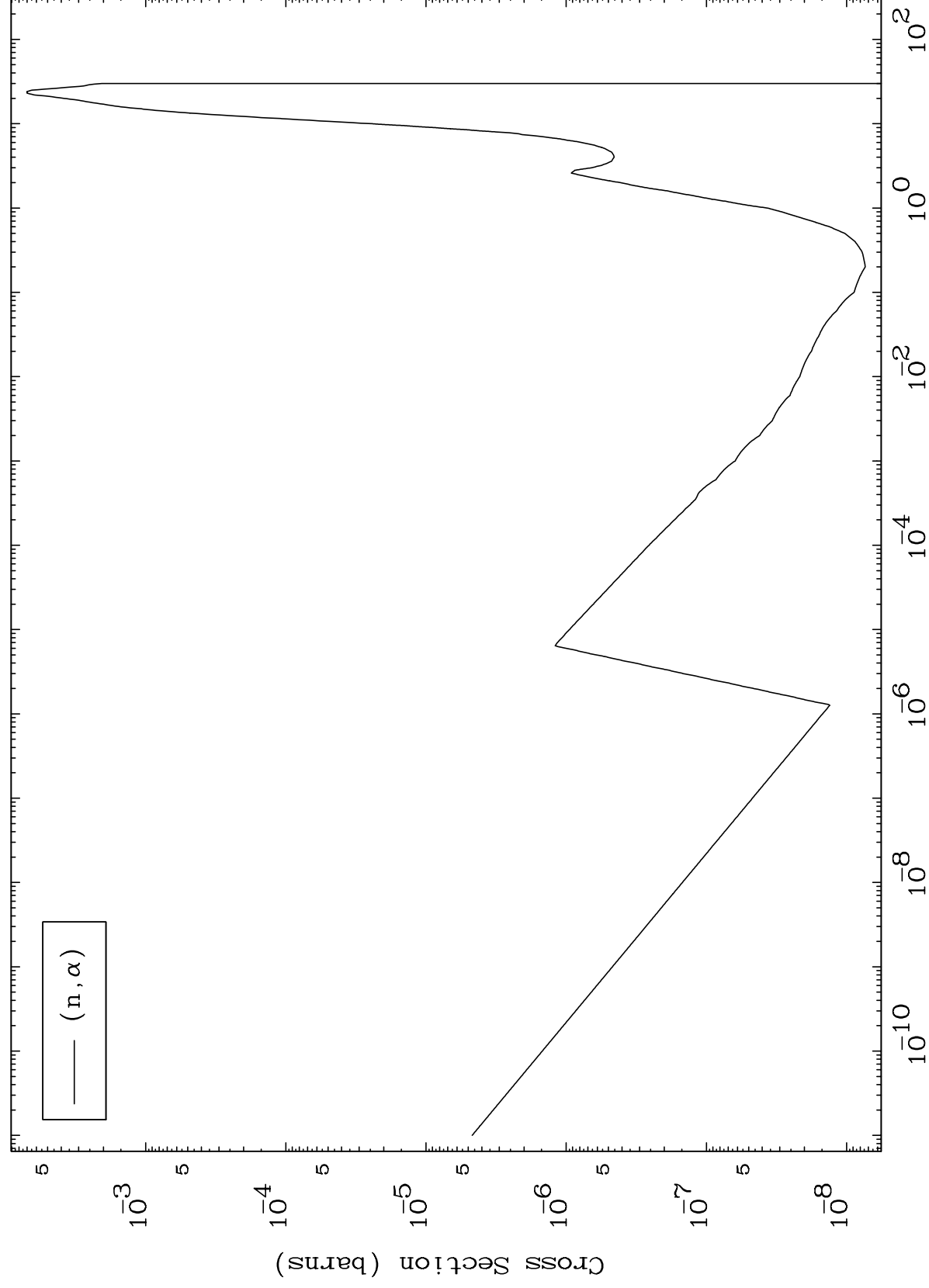
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(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

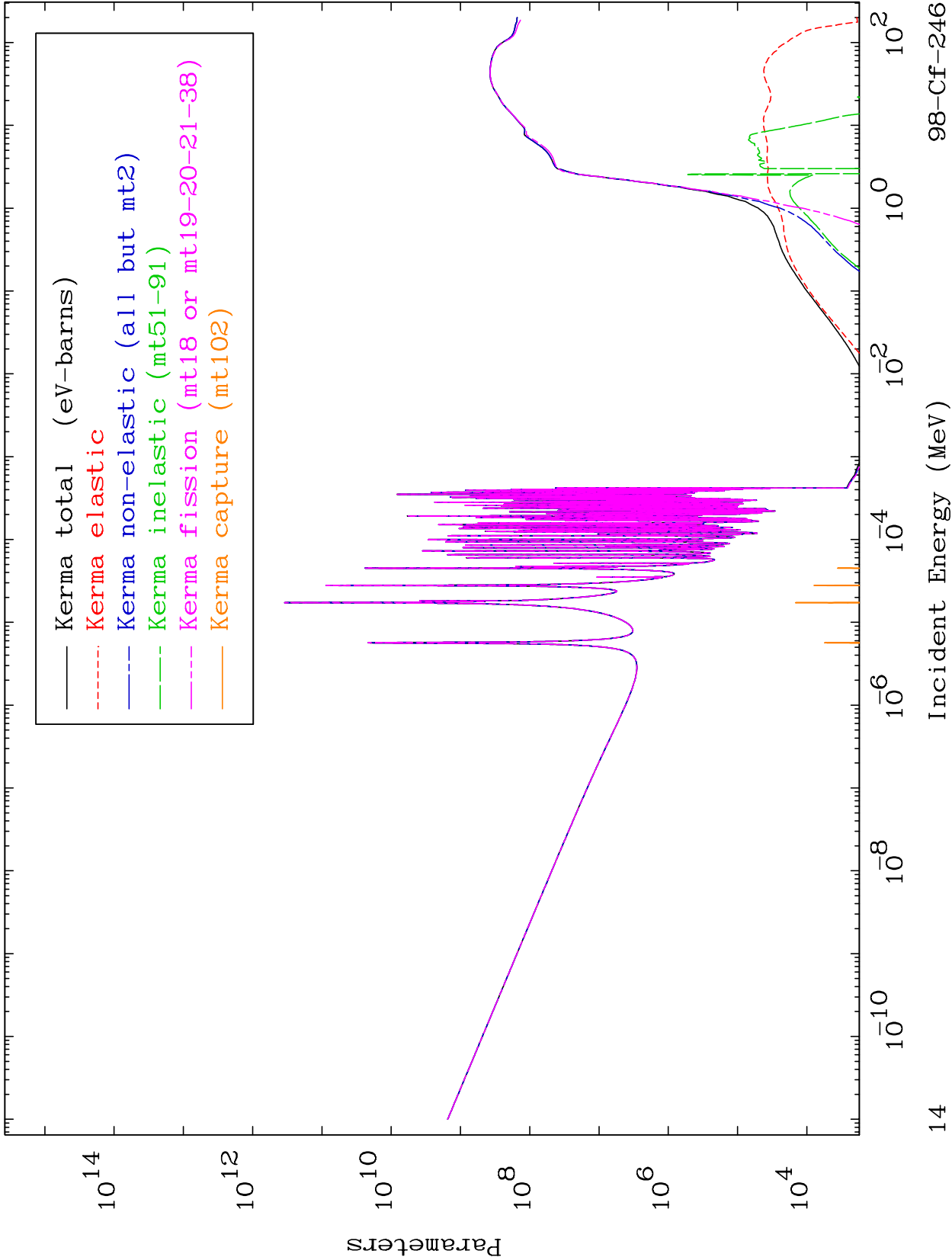
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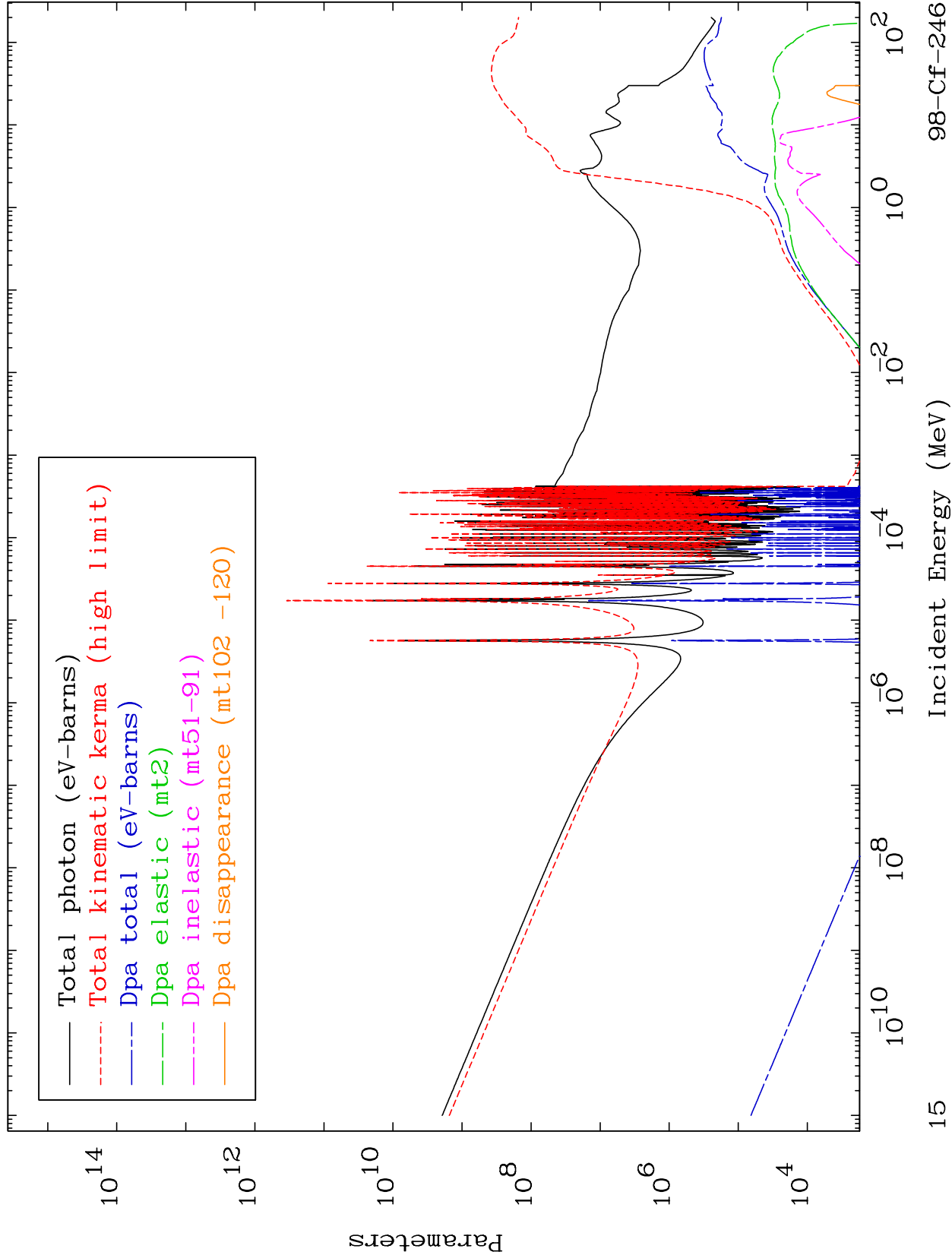


13

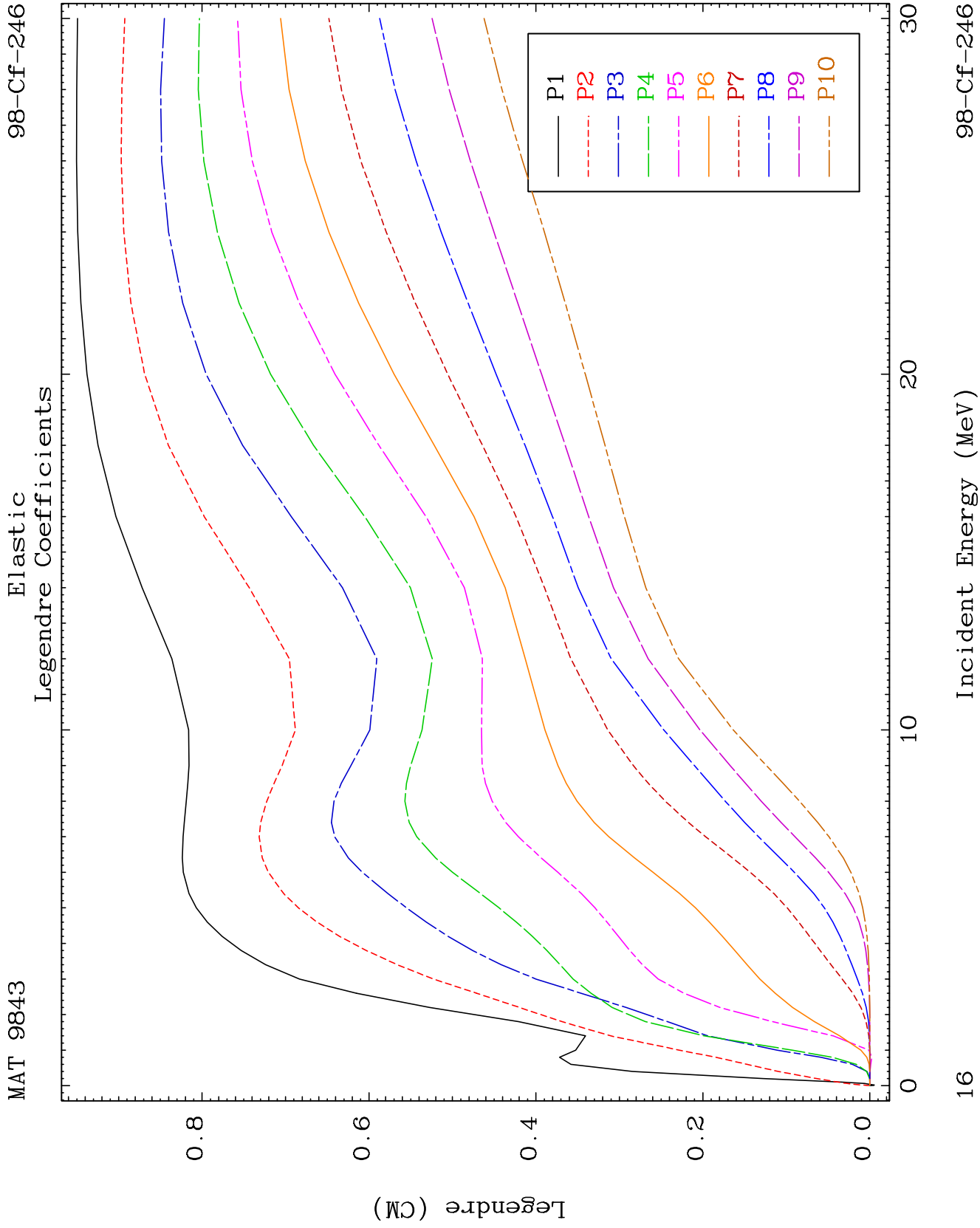
Incident Energy (MeV)

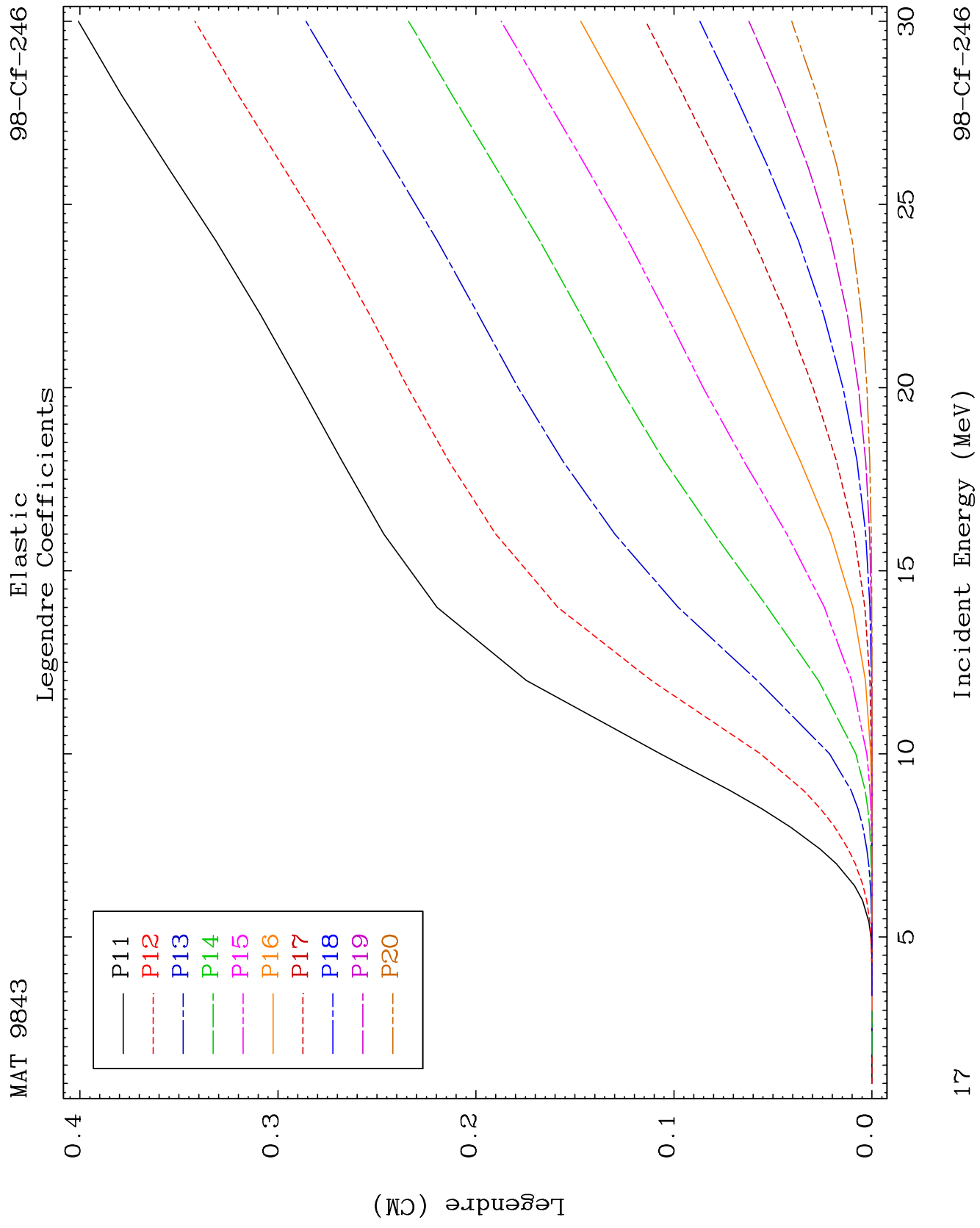
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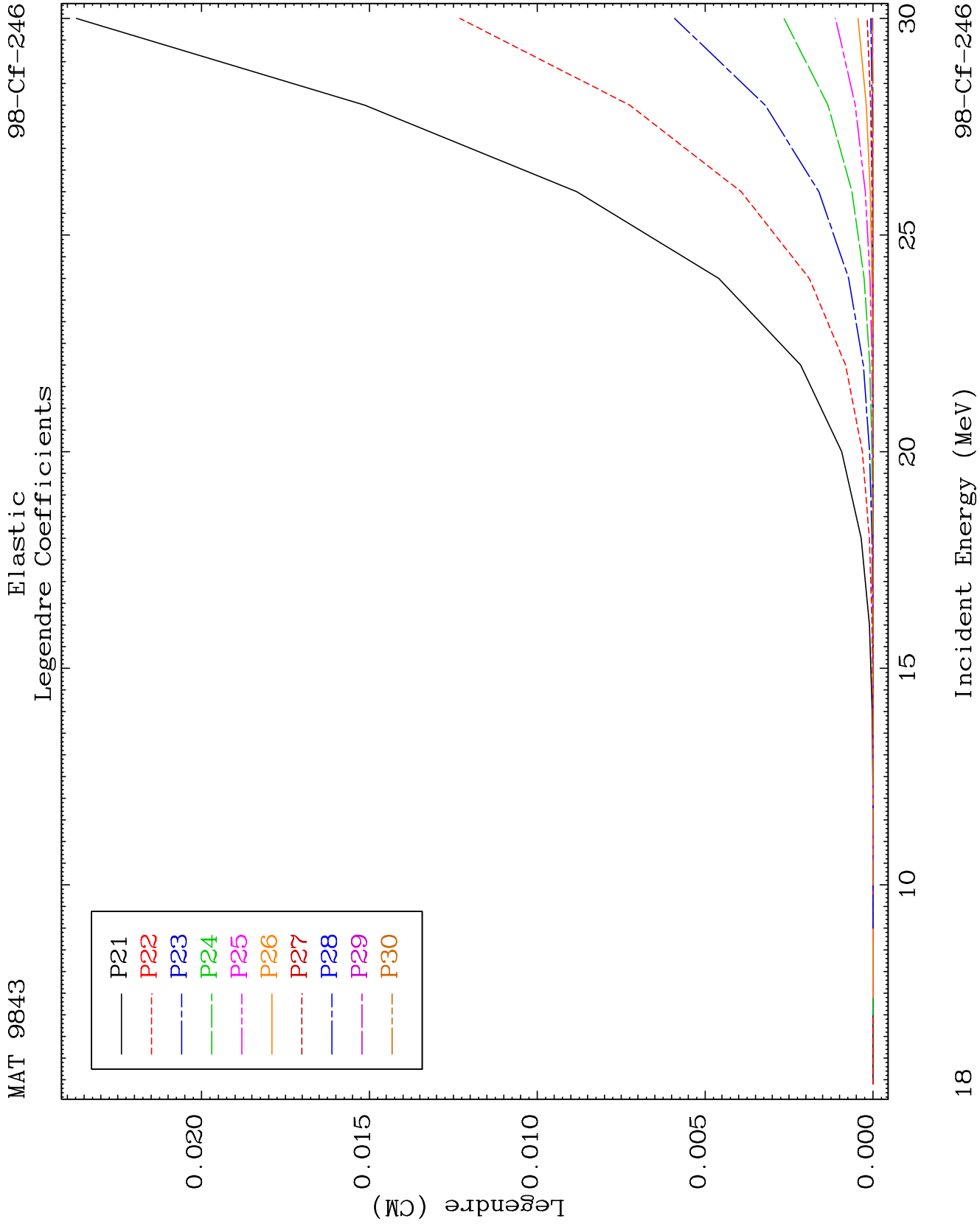


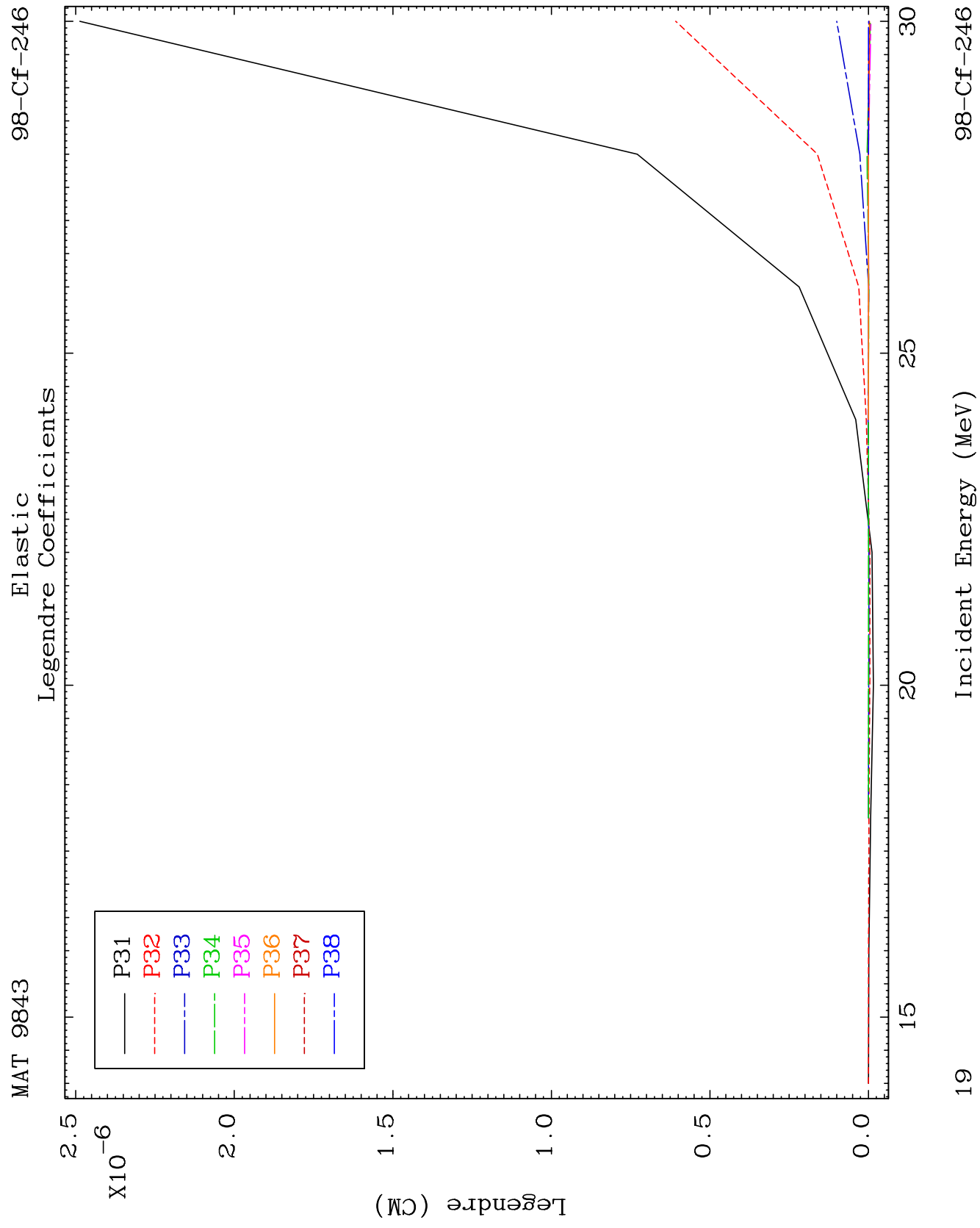








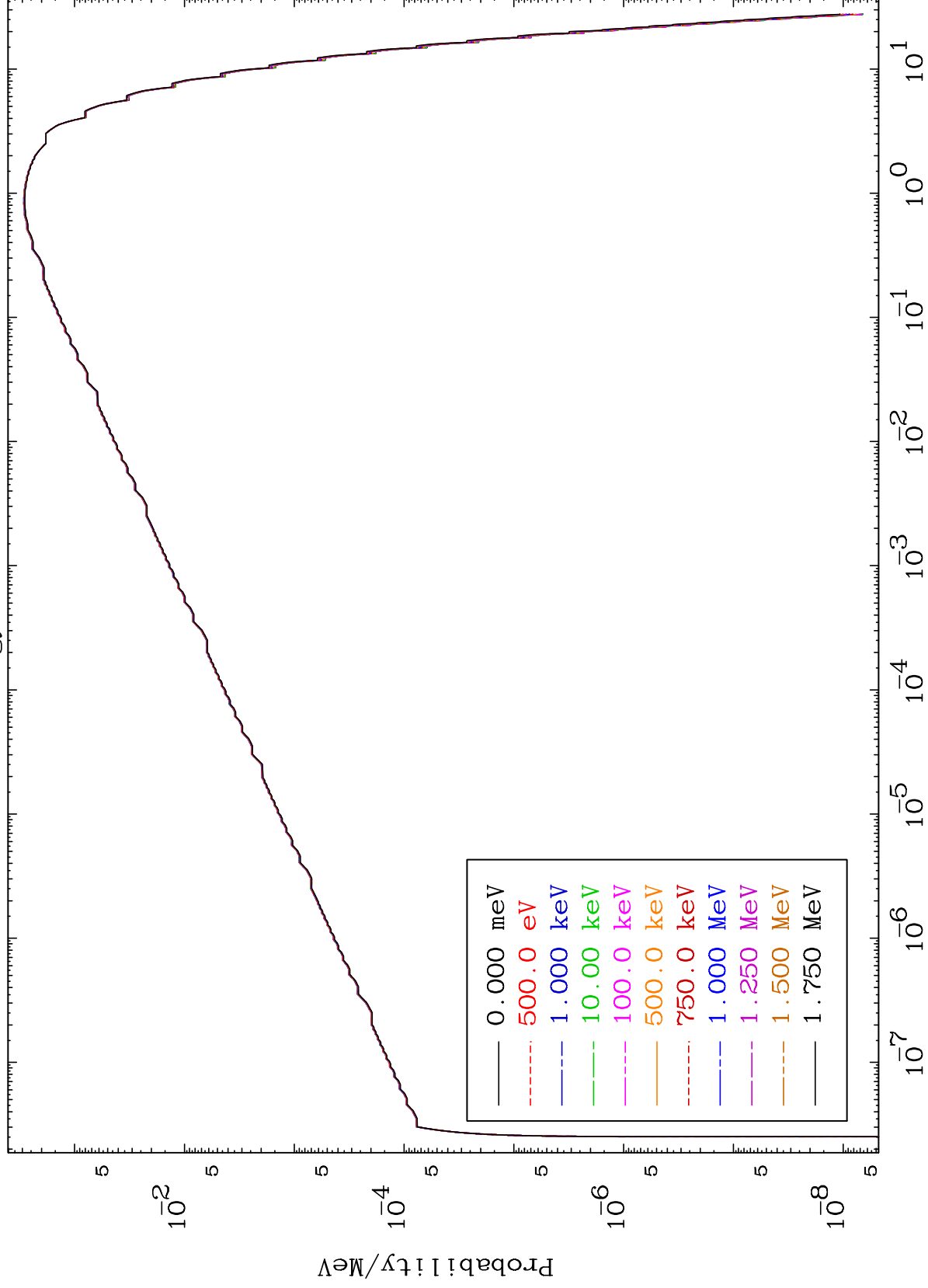




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Fission  
Energy Distribution

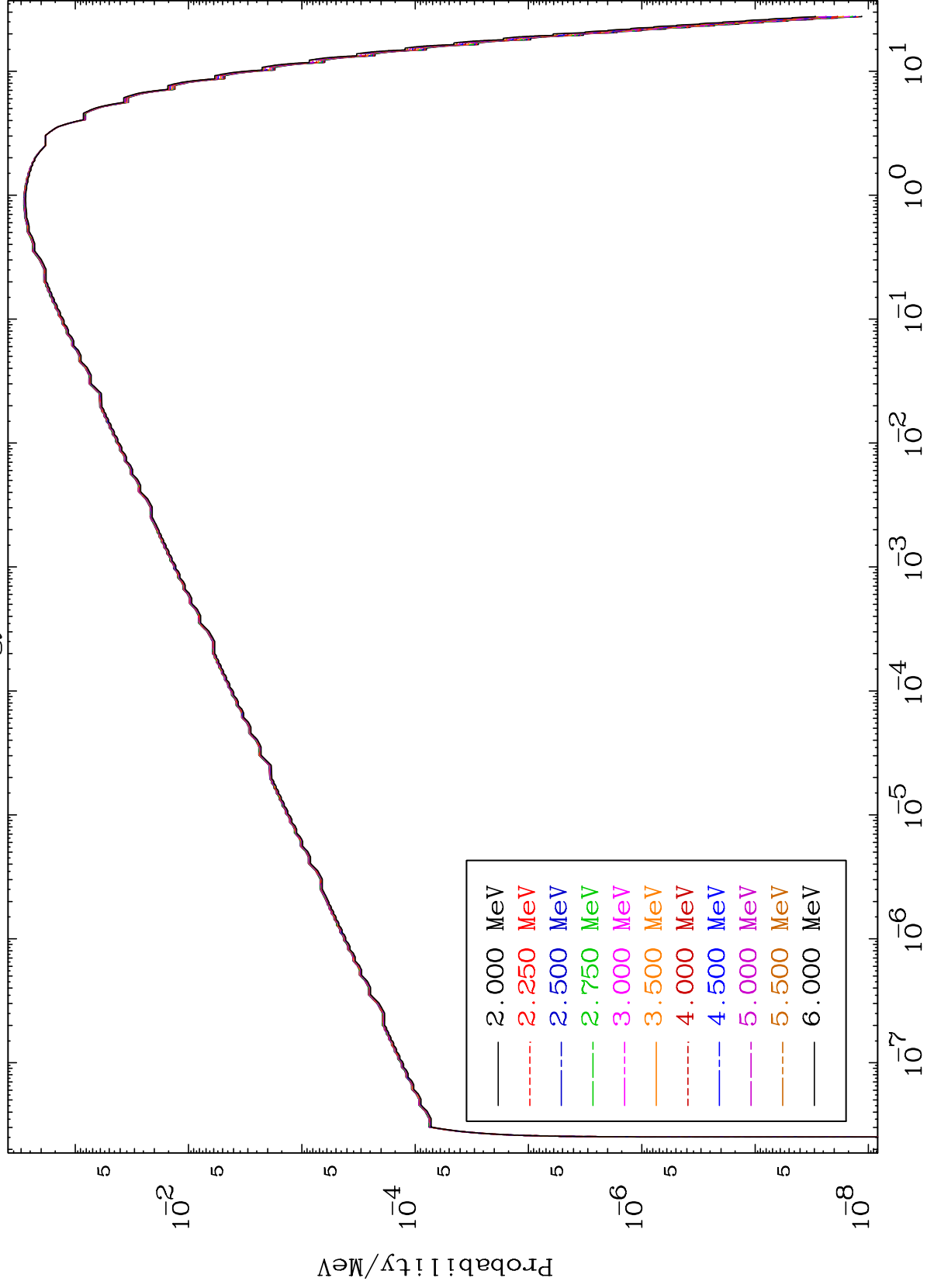
98-Cf-246



MAT 9843

Fission  
Energy Distribution

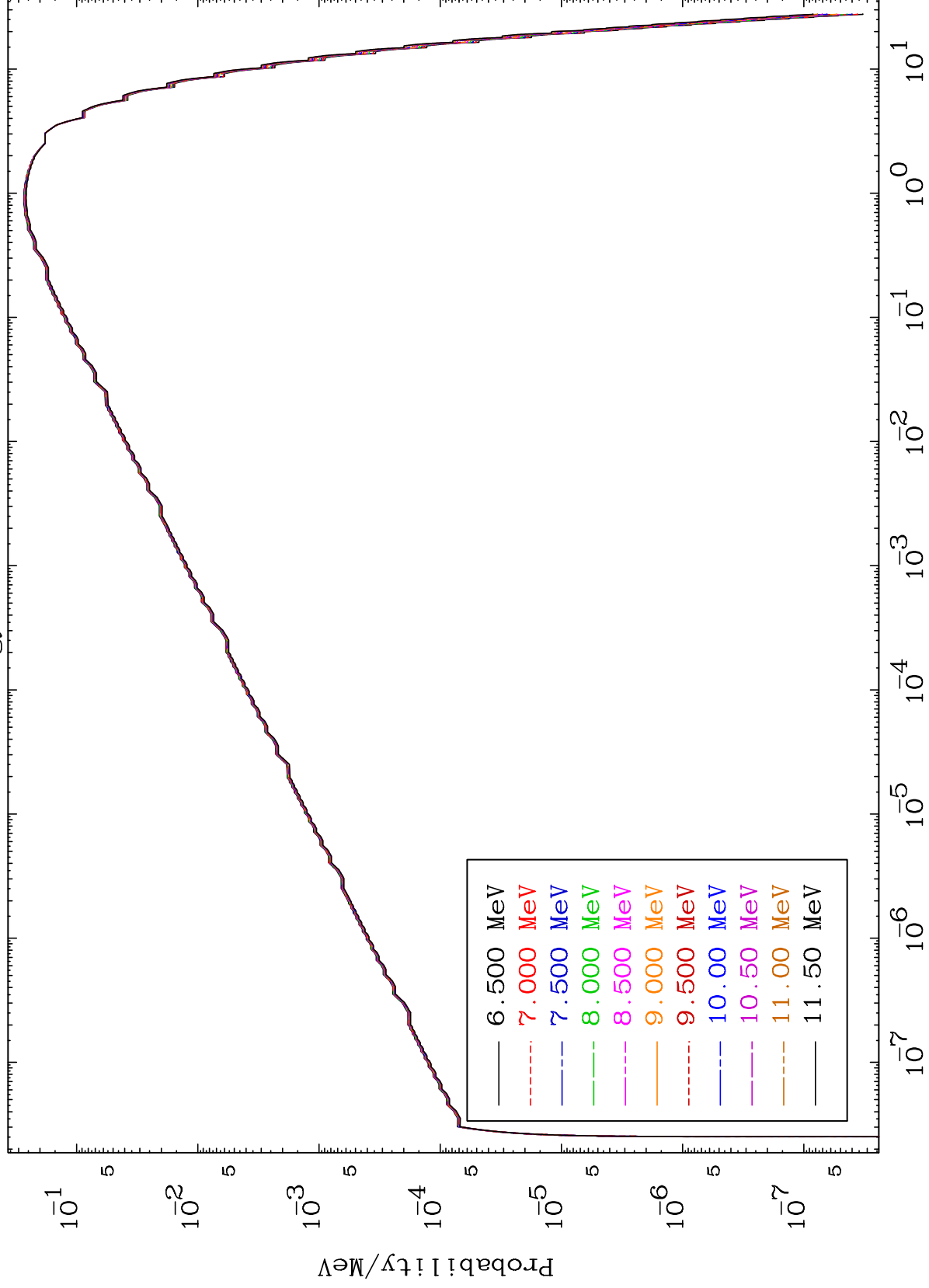
98-Cf-246



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Fission  
Energy Distribution

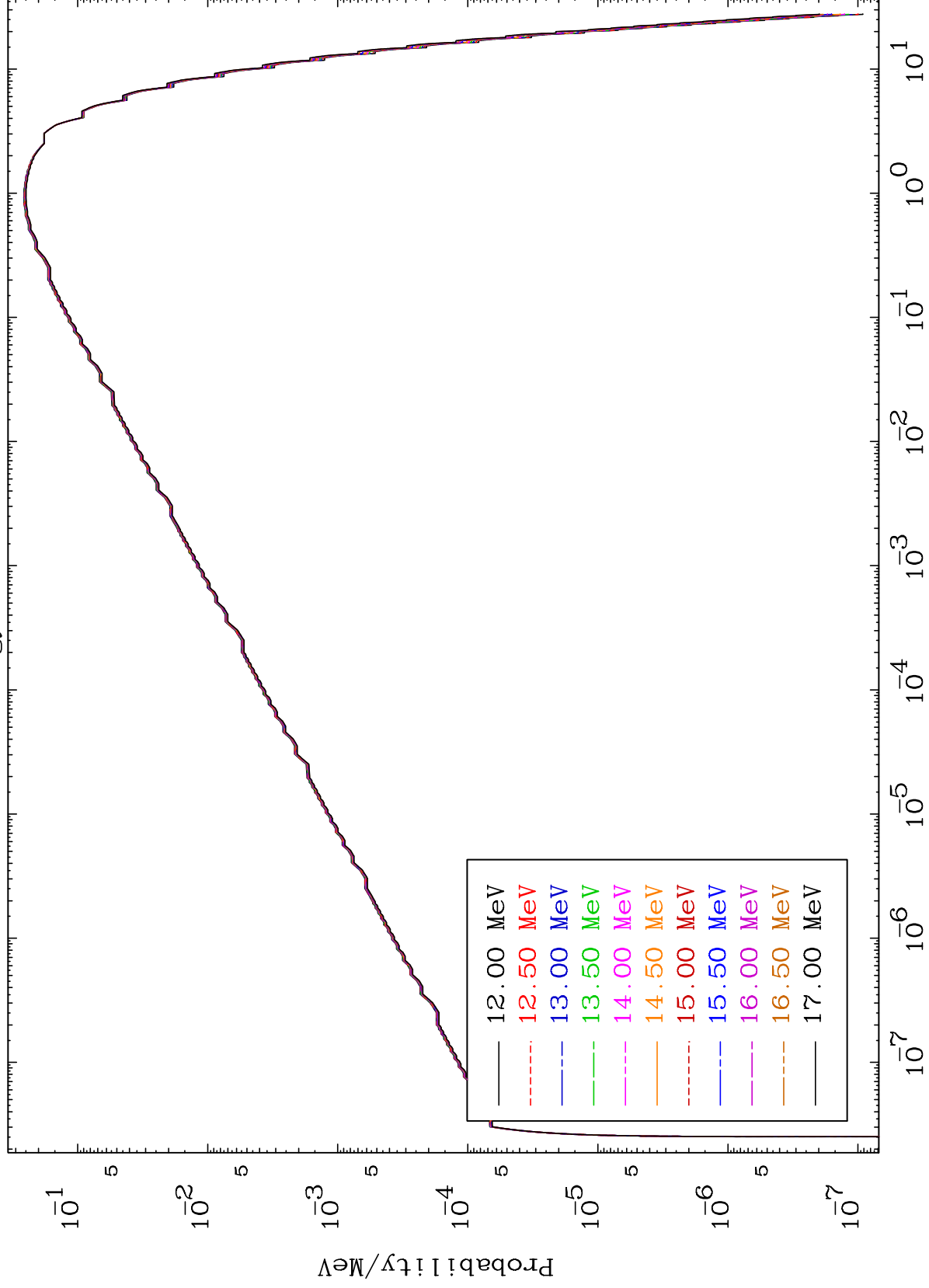
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MAT 9843

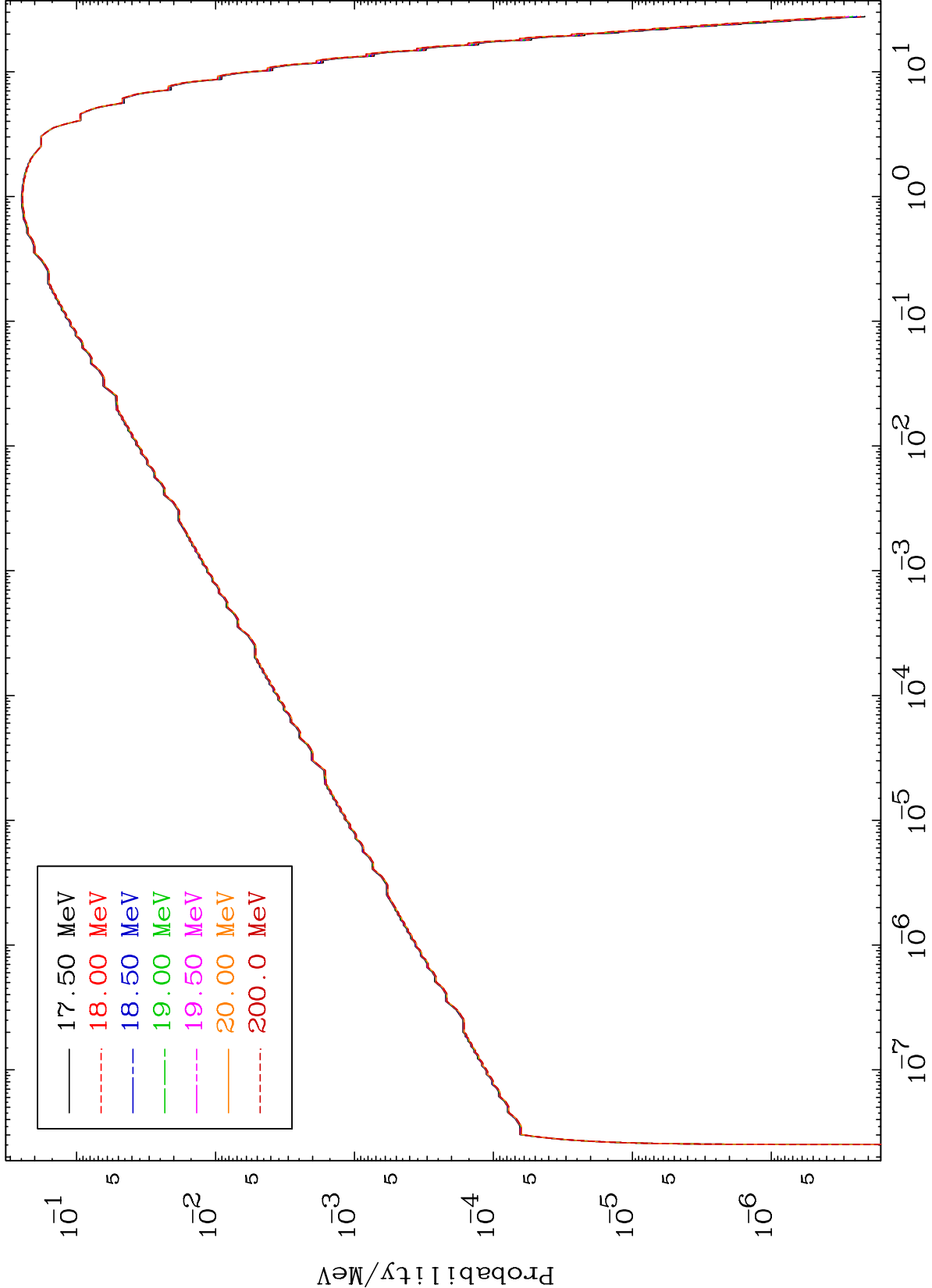
Fission  
Energy Distribution

98-Cf-246





Fission  
Energy Distribution



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(n,2n) 2 $\alpha$

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Radionuclide Production Cross Section

