

Program EVALPLOT
(Version 2017-1)

by

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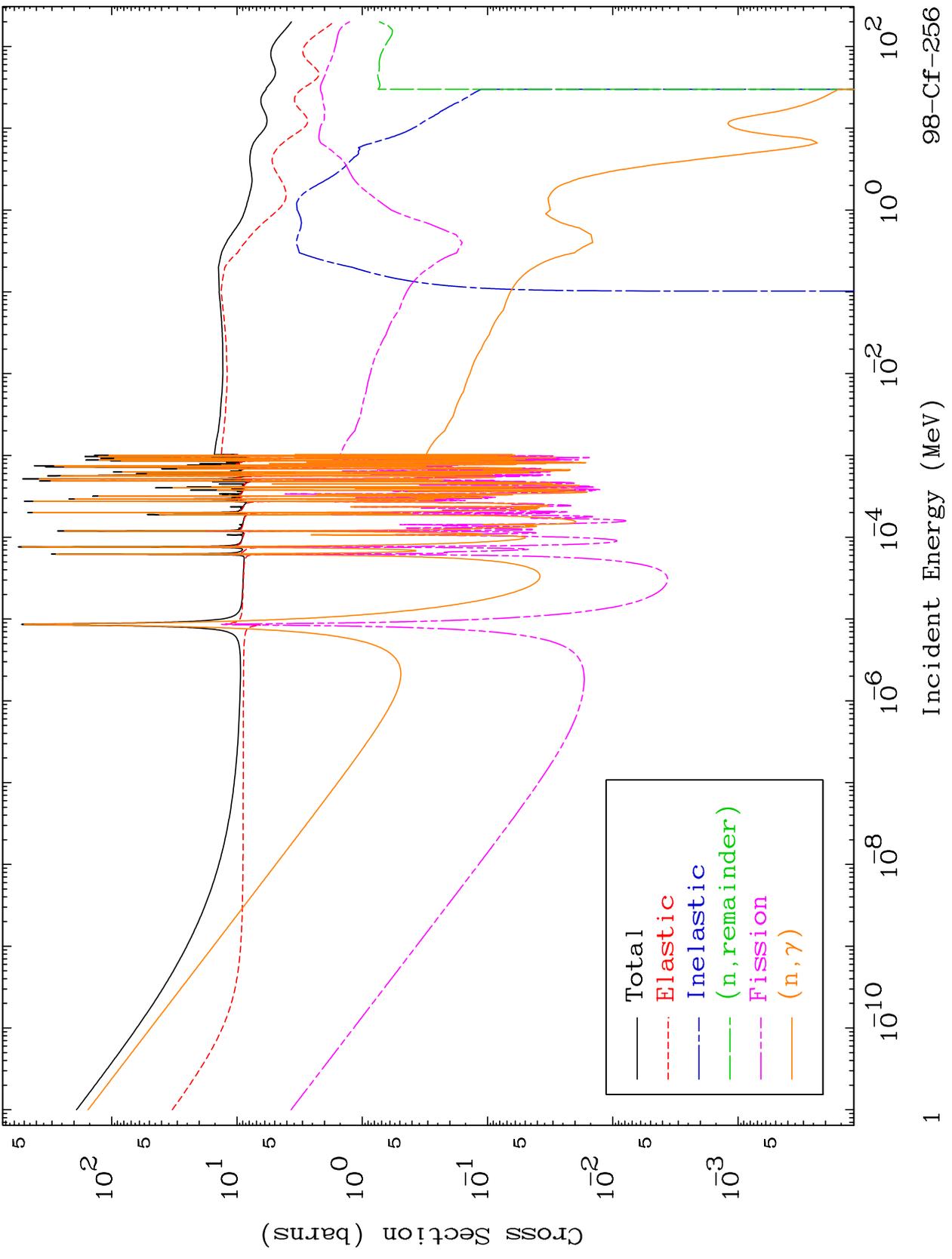
Press Mouse Button to Start

MAT 9873

Major

293 Kelvin Cross Sections

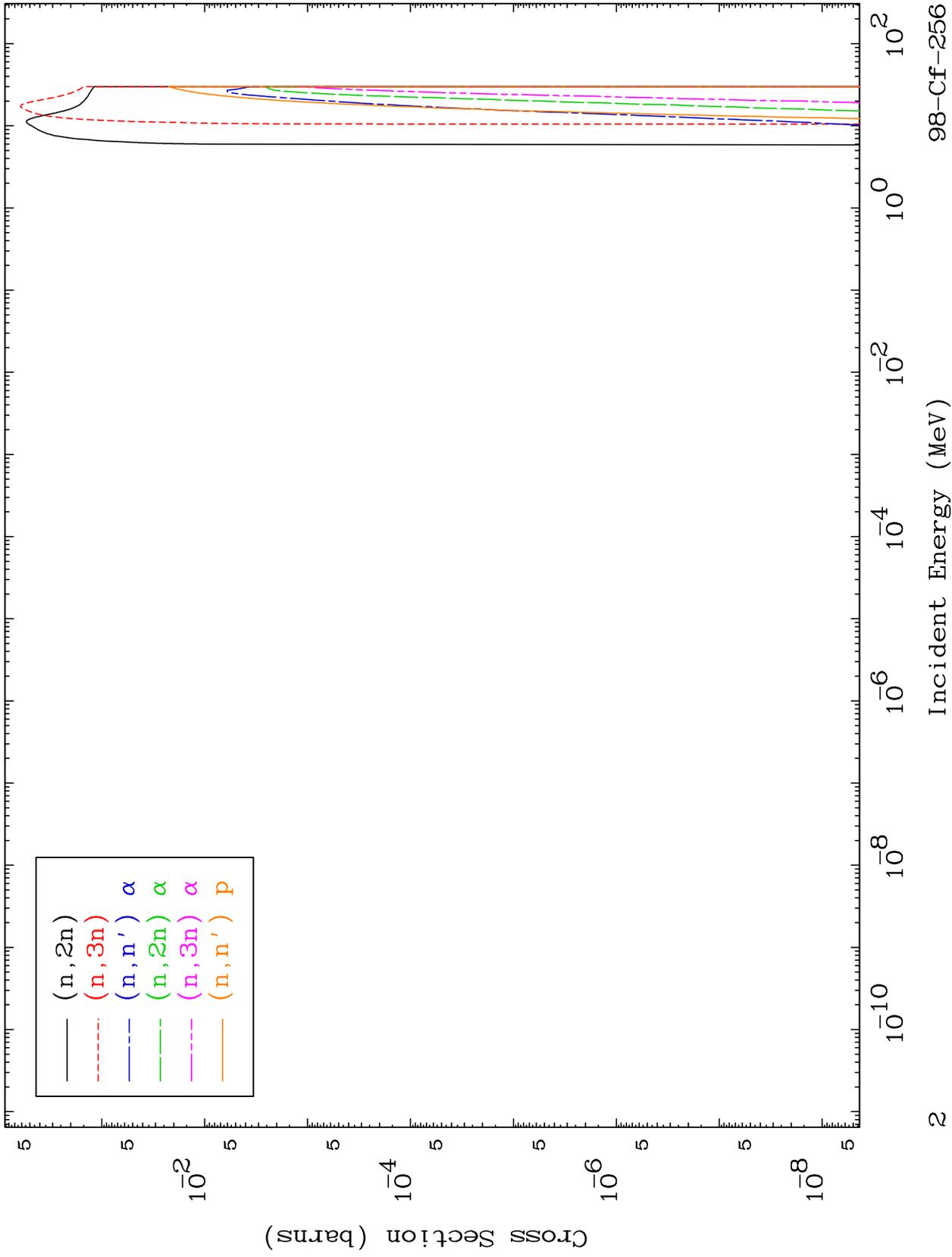
98-Cf-256



MAT 9873

Neutron Production
293 Kelvin Cross Sections

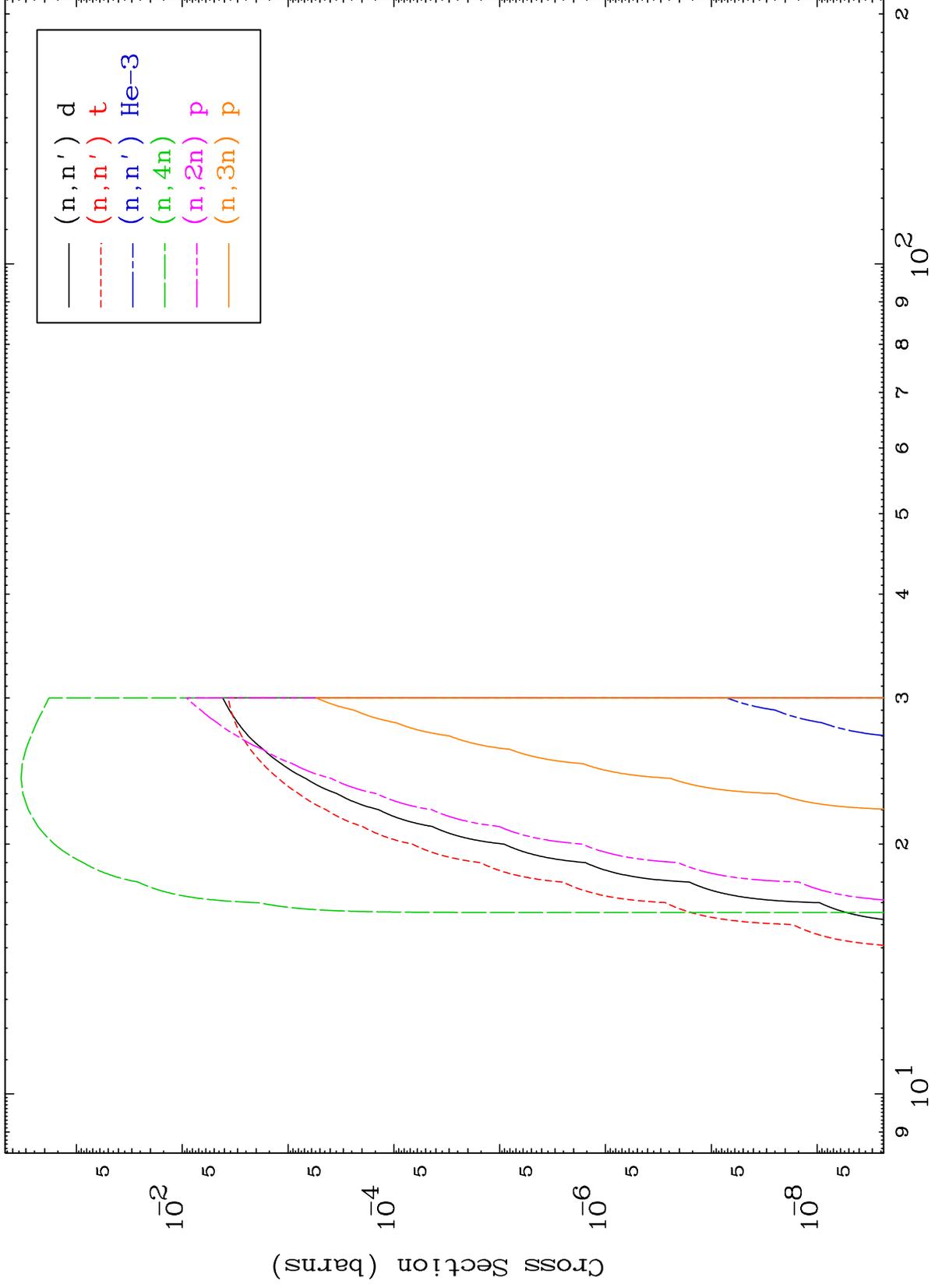
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Neutron Production
293 Kelvin Cross Sections

98-Cf-256



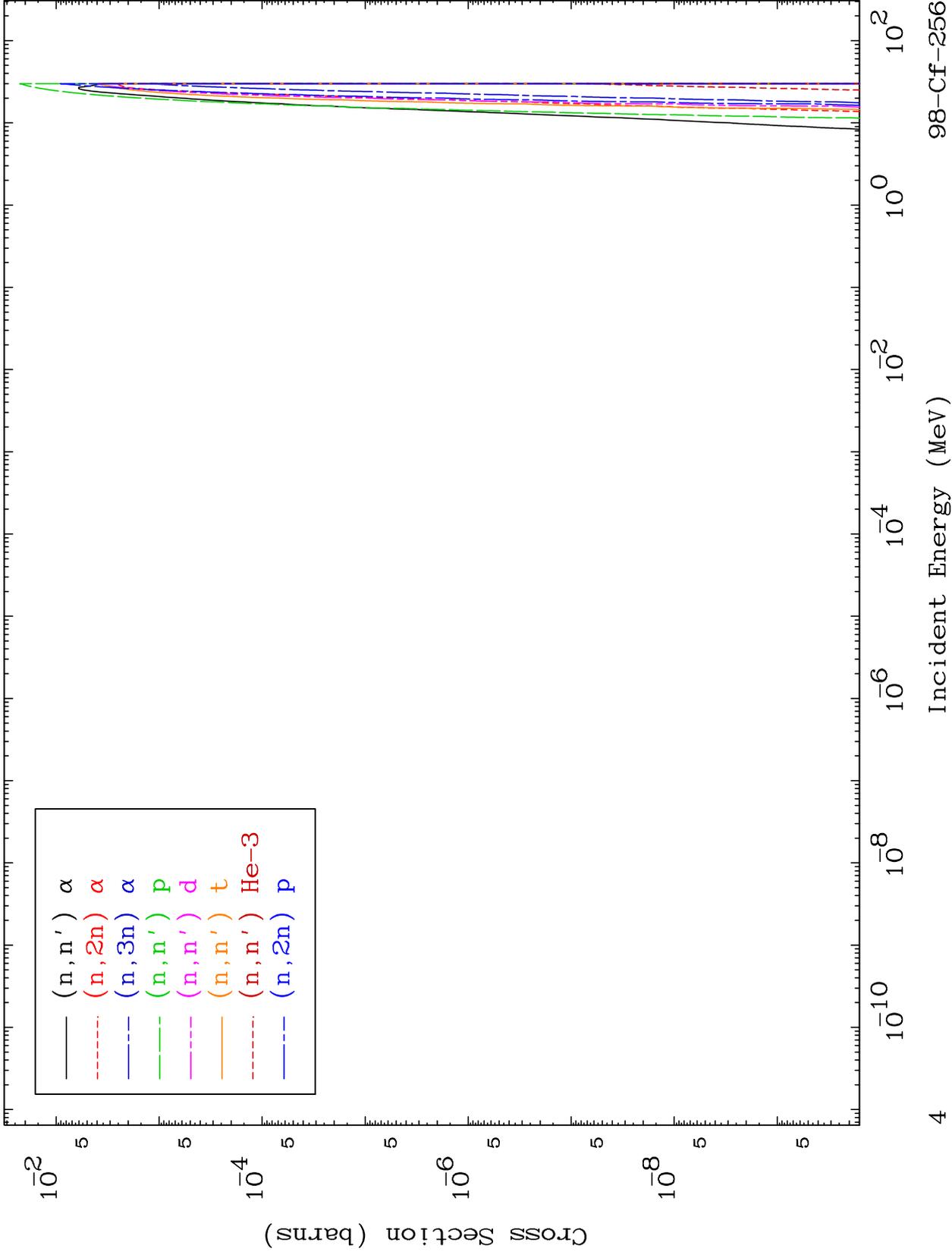
Incident Energy (MeV)

98-Cf-256

MAT 9873

Charged Particle
293 Kelvin Cross Sections

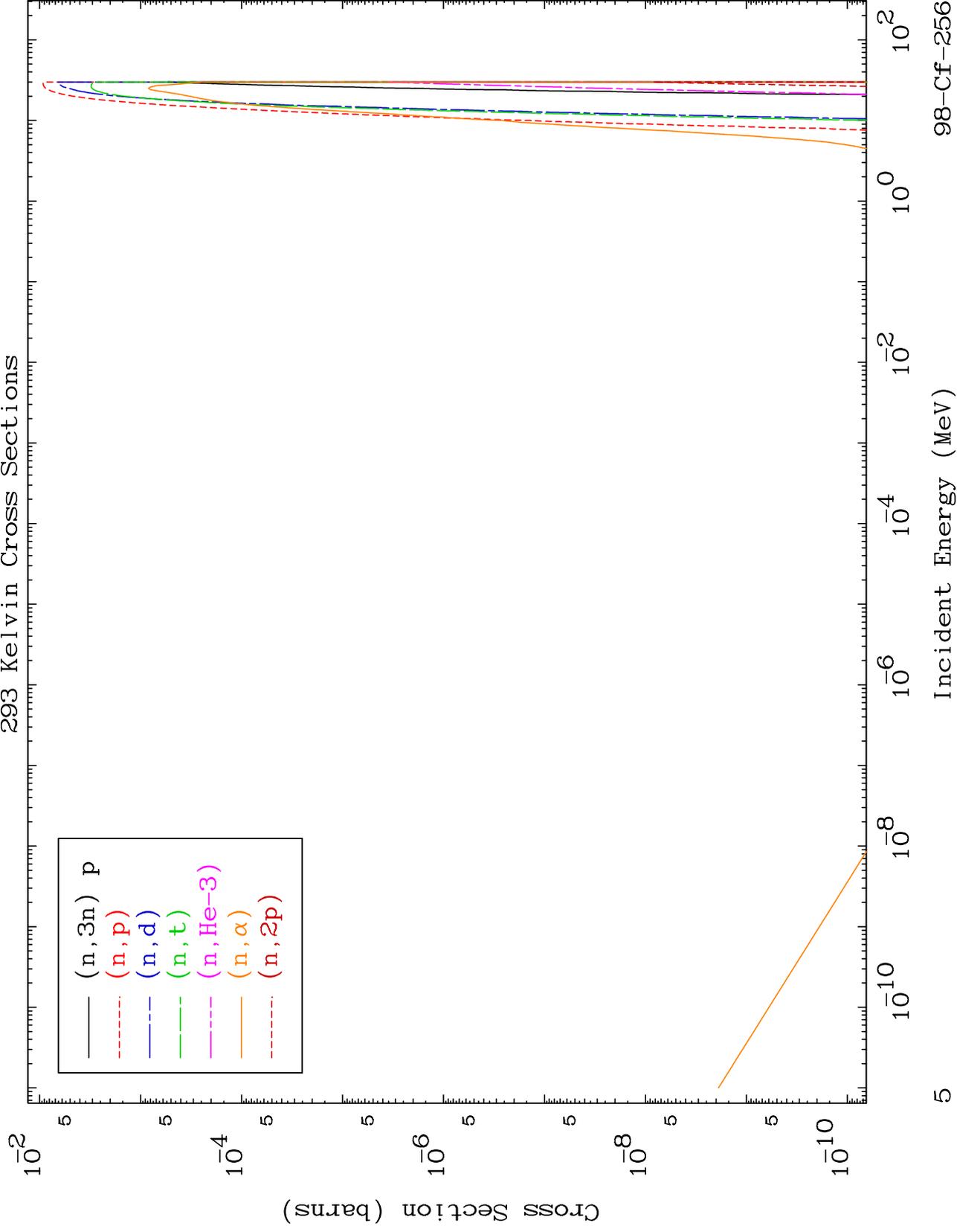
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Charged Particle
293 Kelvin Cross Sections

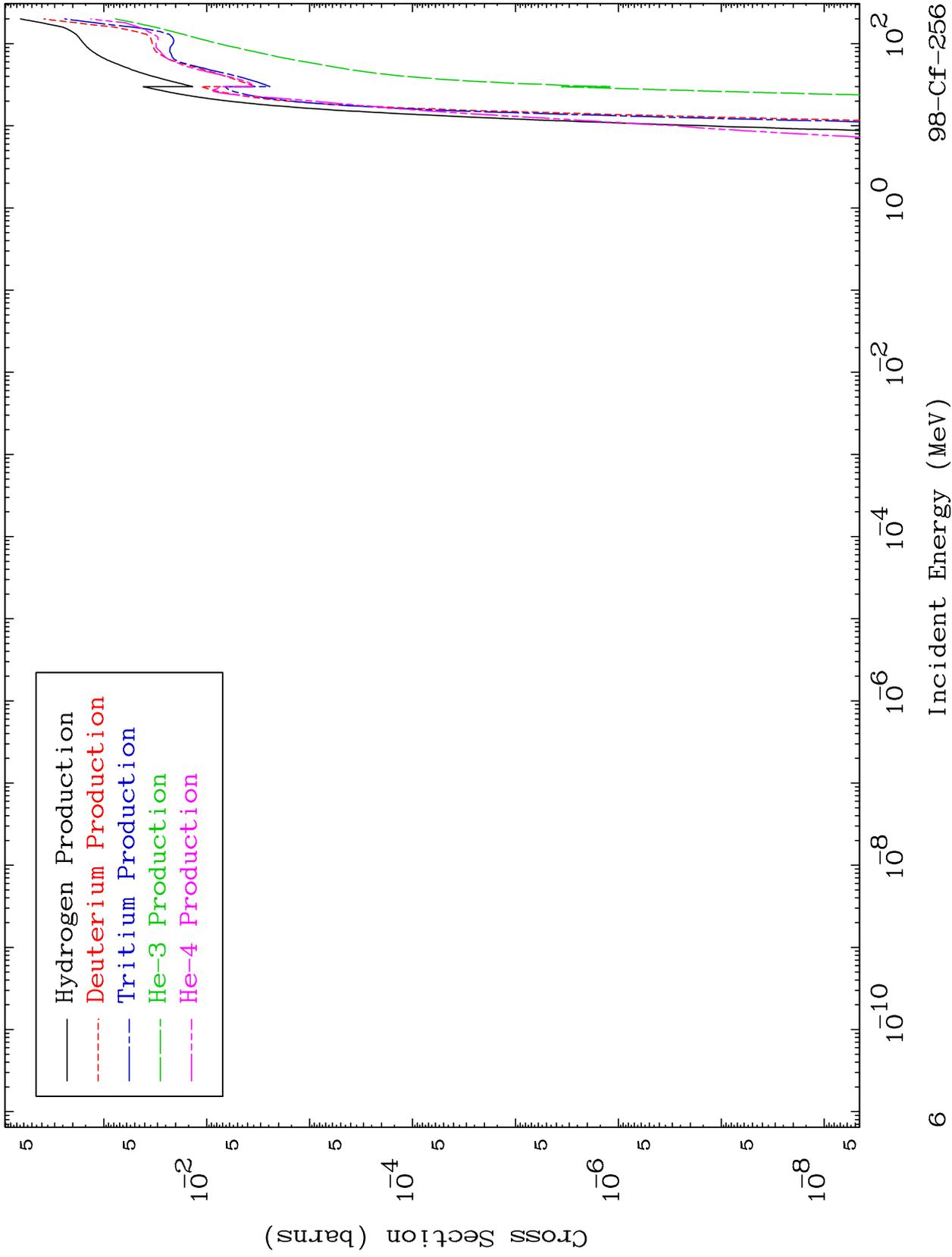
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Particle Production
293 Kelvin Cross Sections

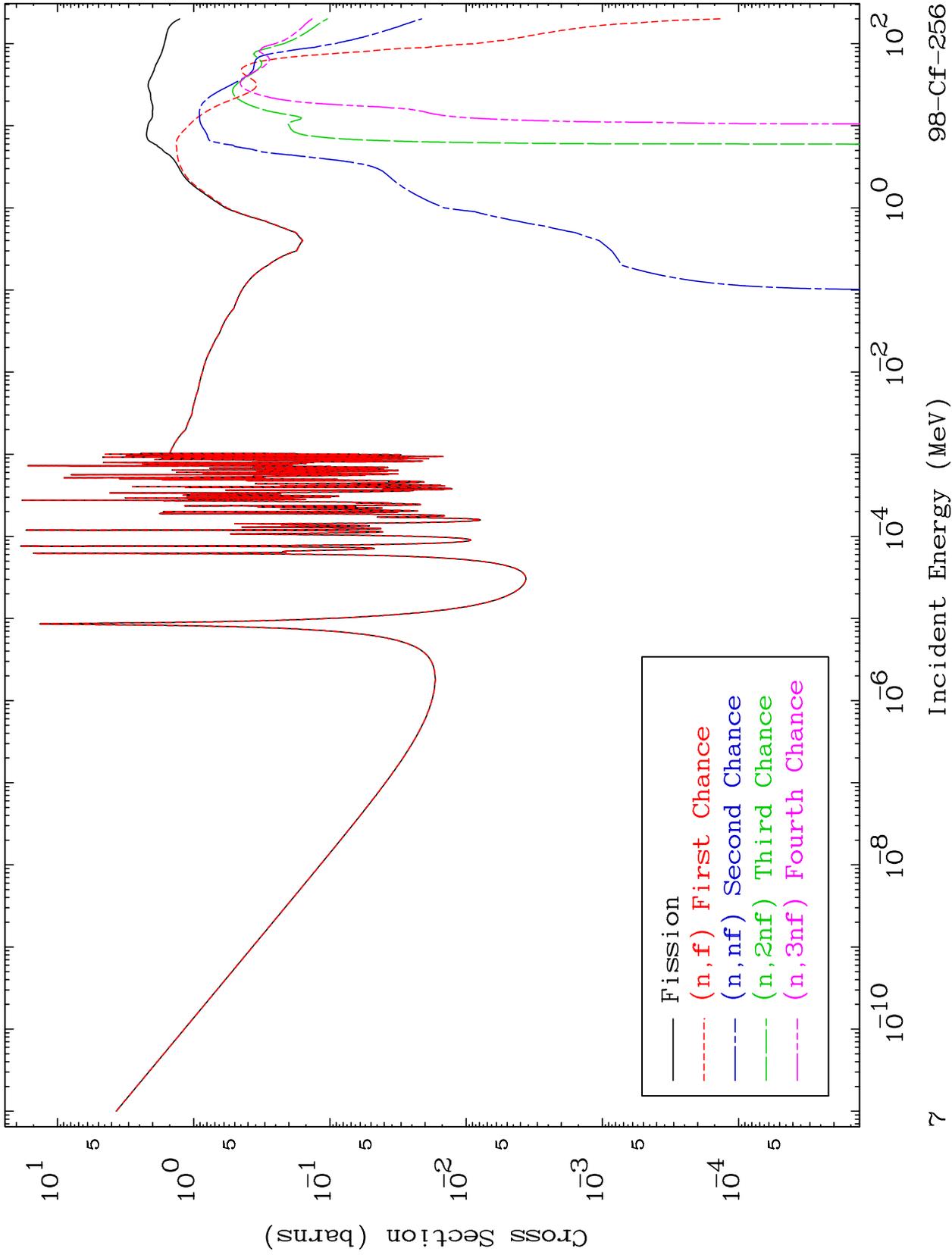
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Fission
293 Kelvin Cross Sections

98-Cf-256

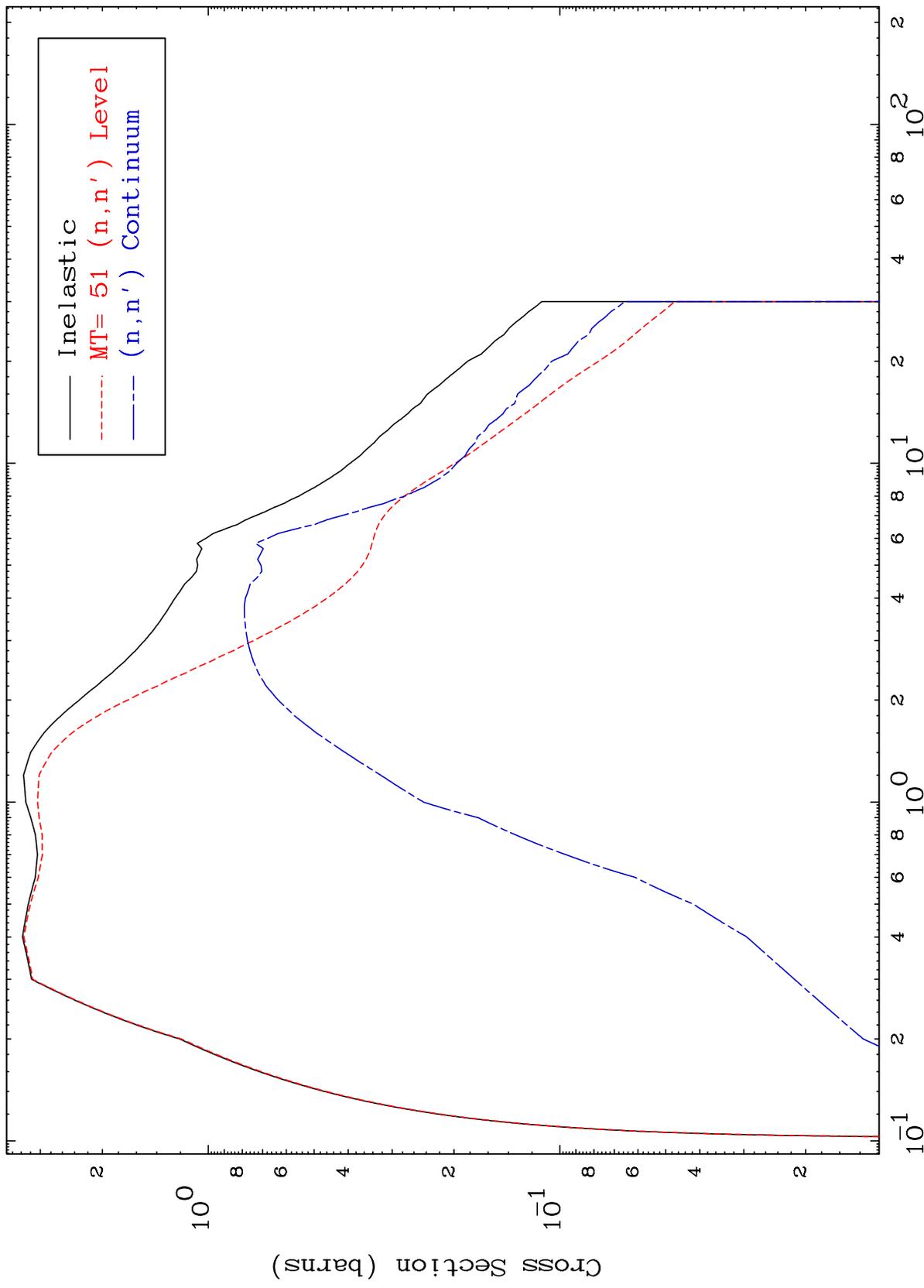


MAT 9873

(n,n') Level

98-Cf-256

293 Kelvin Cross Sections



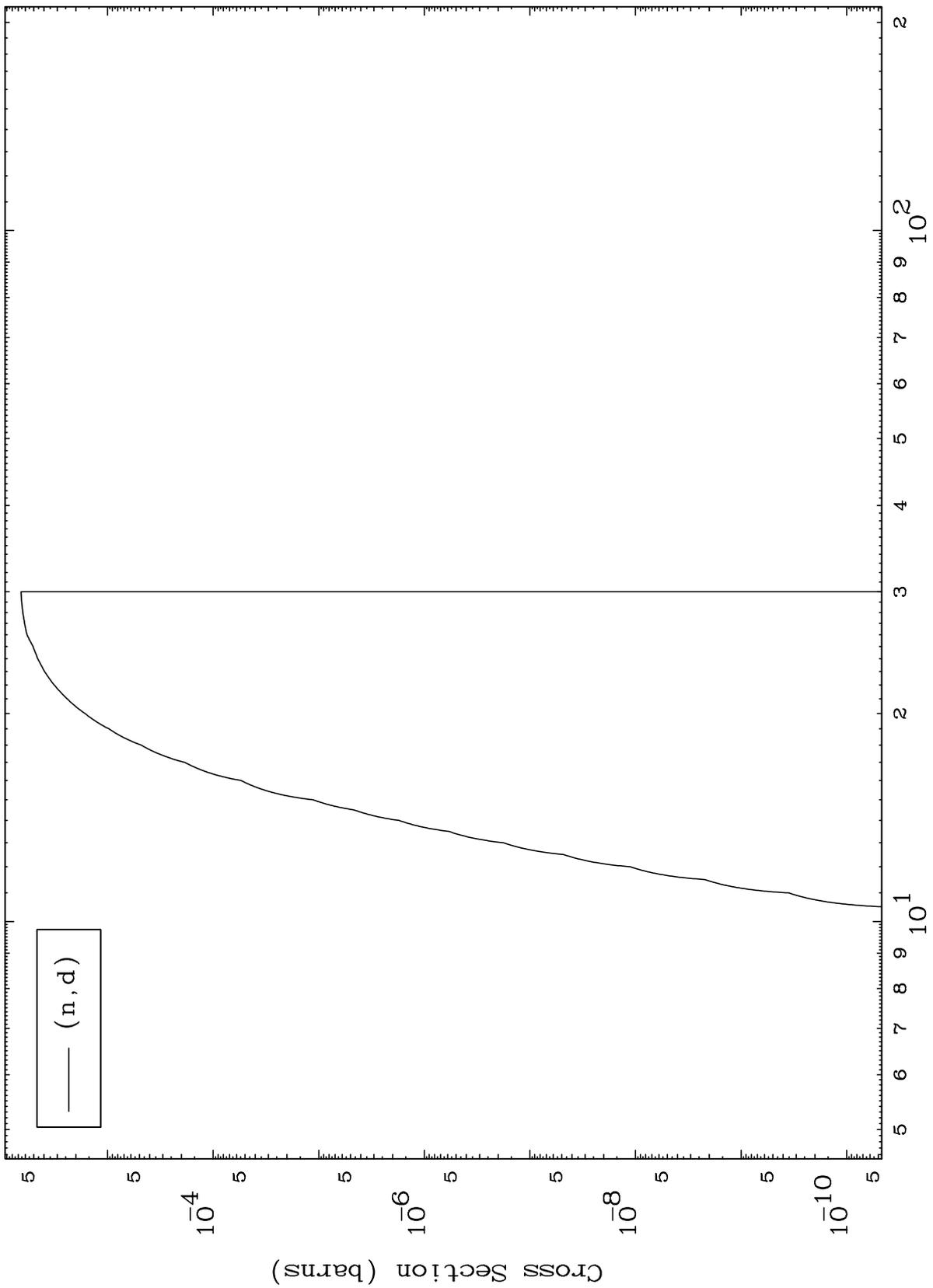
Incident Energy (MeV)

98-Cf-256

MAT 9873

(n,d) Levels
293 Kelvin Cross Sections

98-Cf-256



10

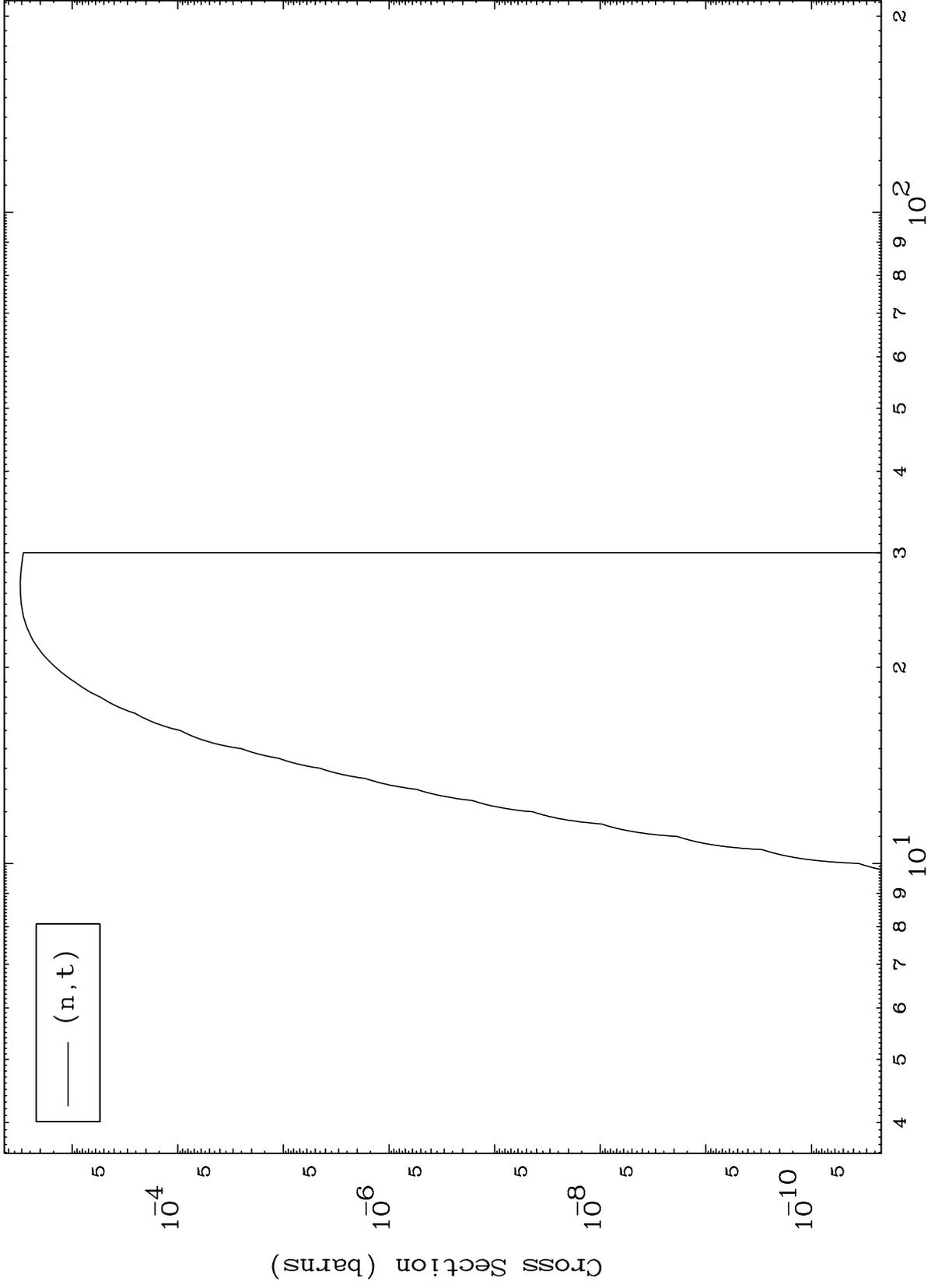
Incident Energy (MeV)

98-Cf-256

MAT 9873

(n,t) Levels
293 Kelvin Cross Sections

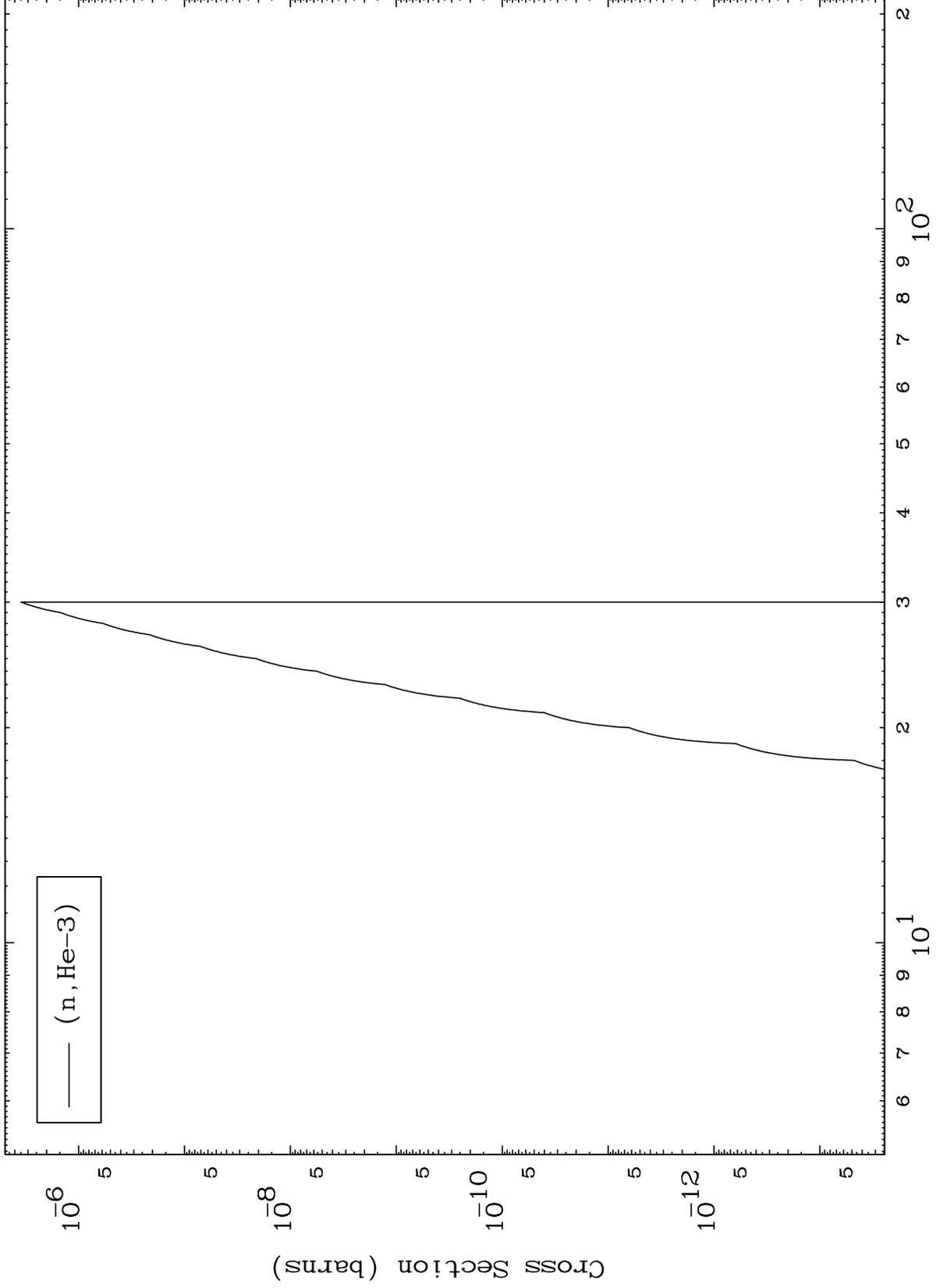
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(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-256



12

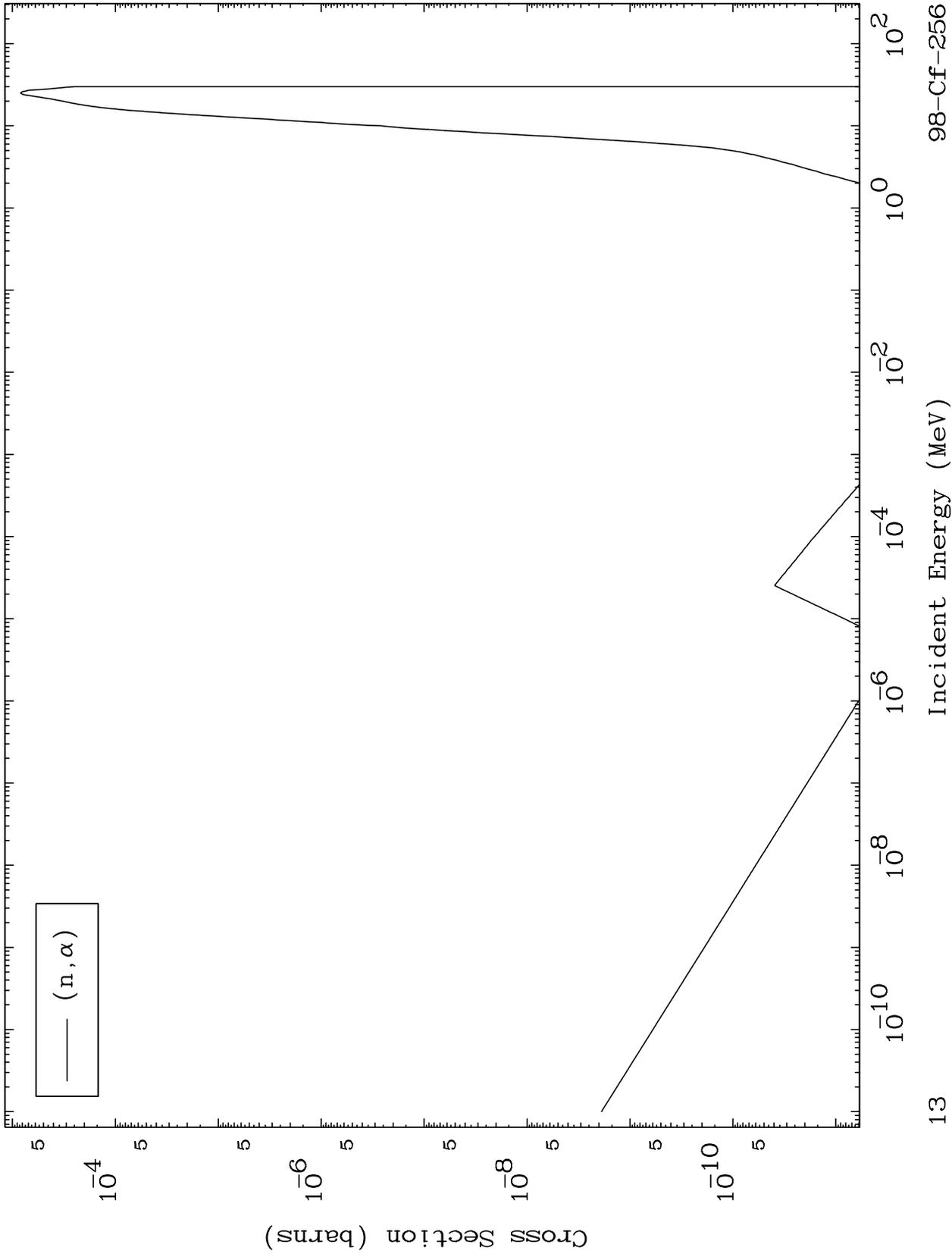
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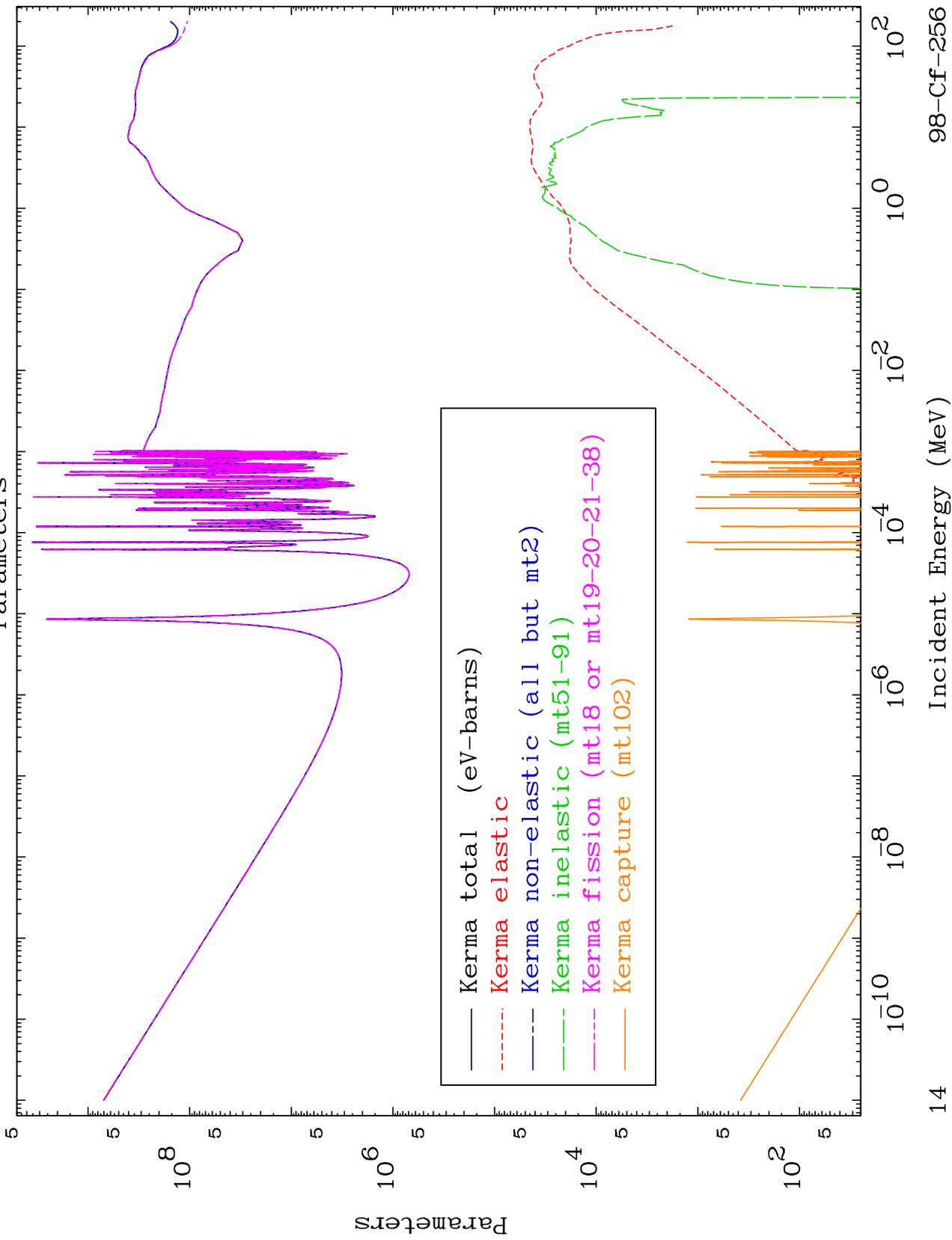
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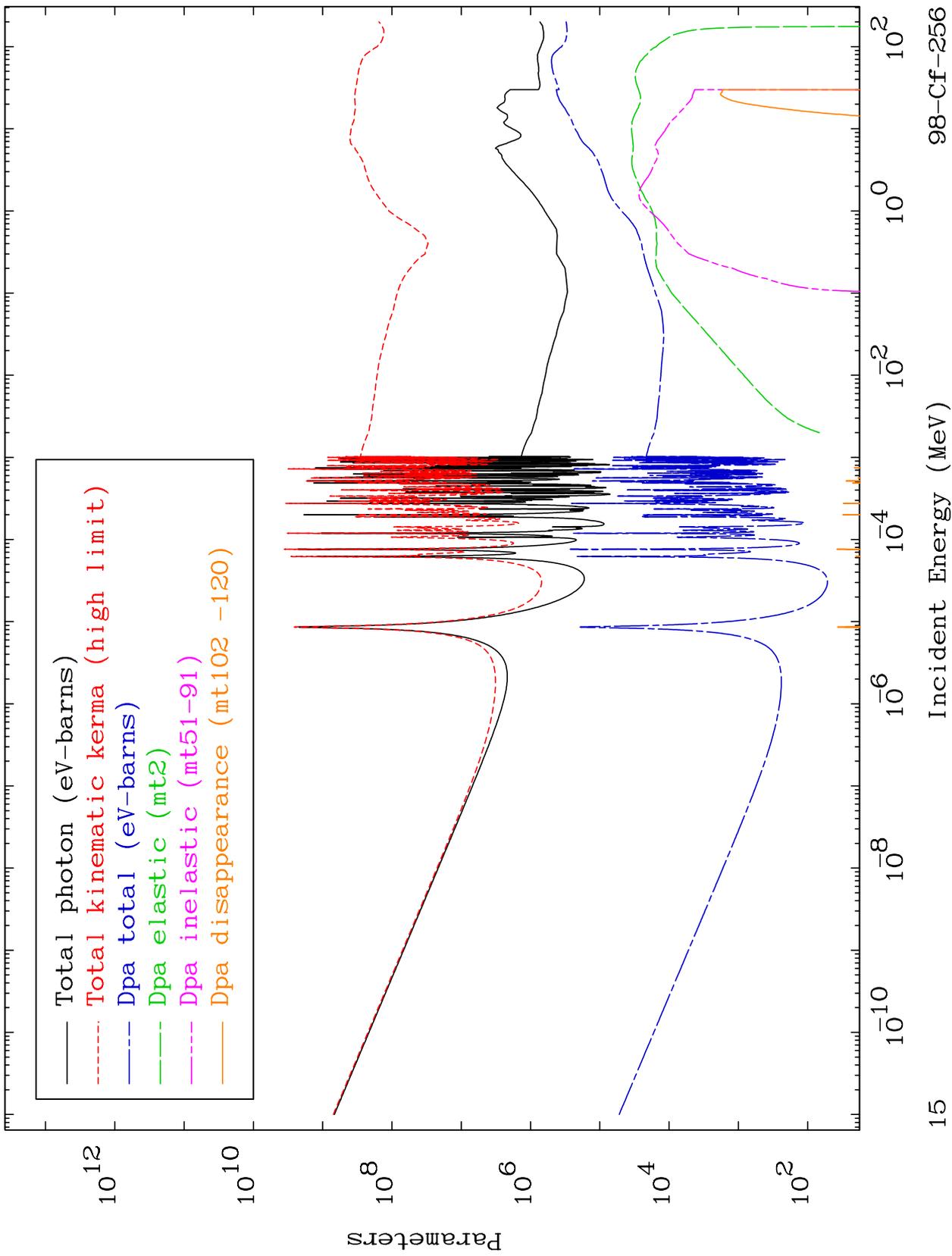
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(n, α) Levels
293 Kelvin Cross Sections

98-Cf-256



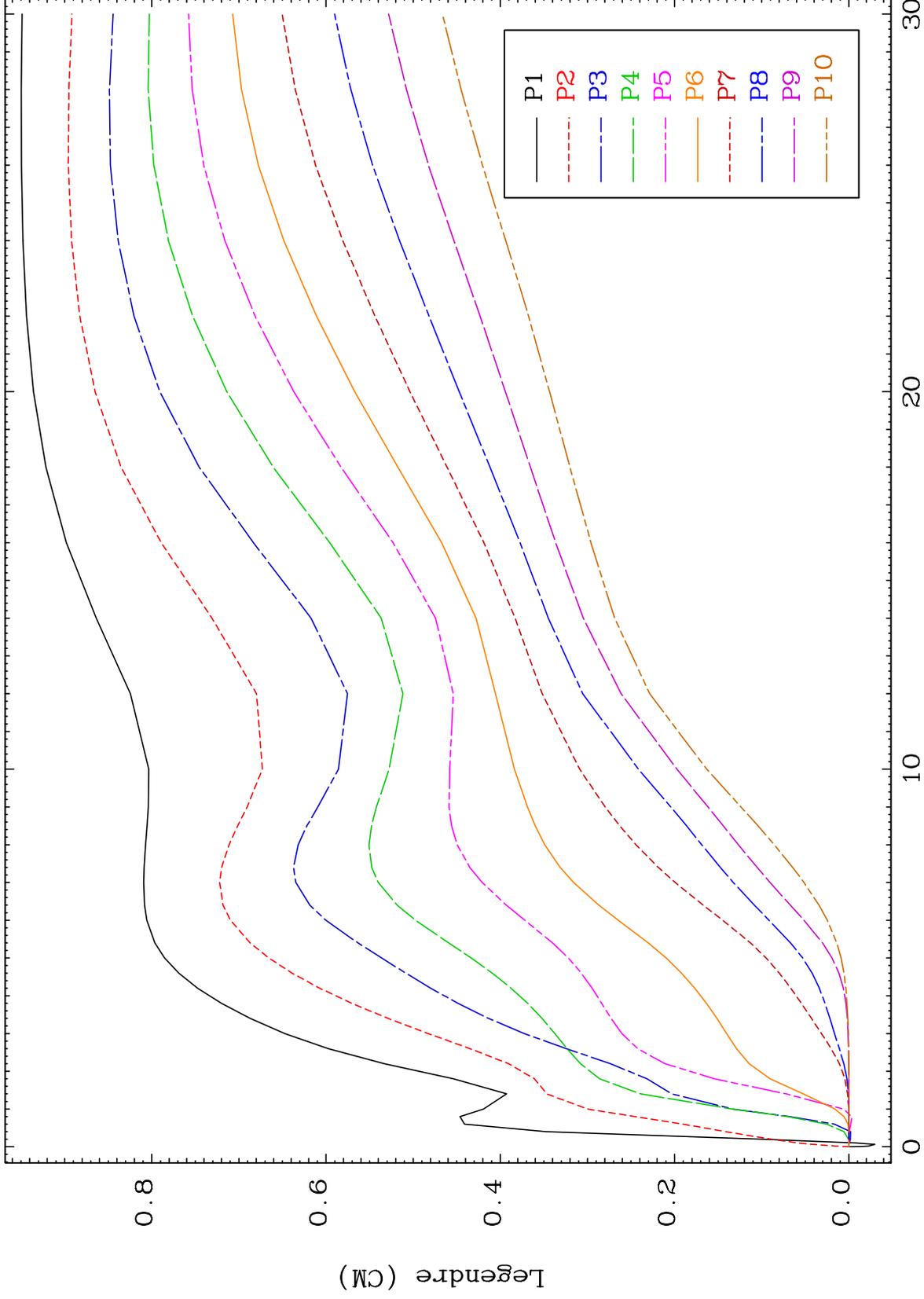




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Elastic Legendre Coefficients

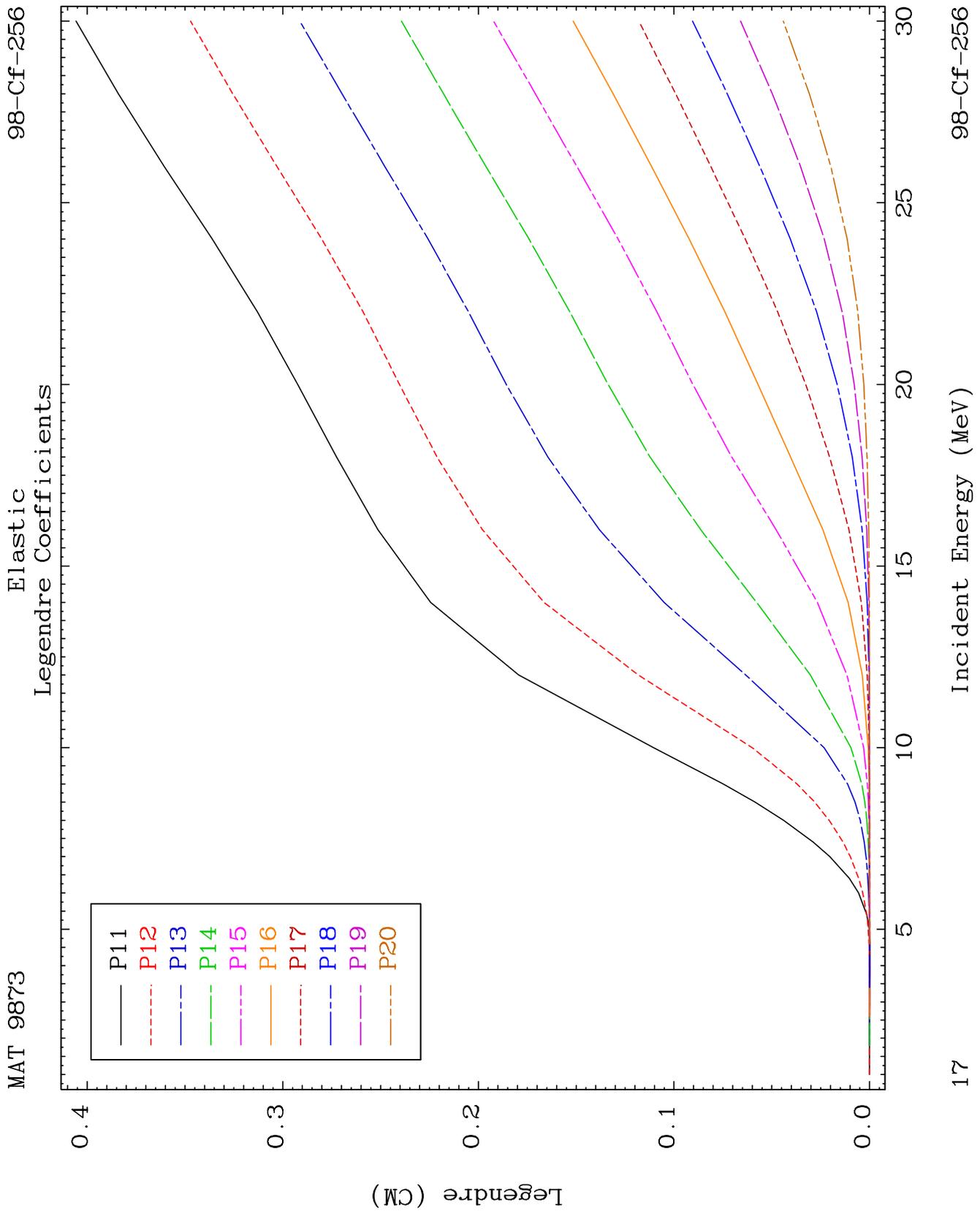
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16

Incident Energy (MeV)

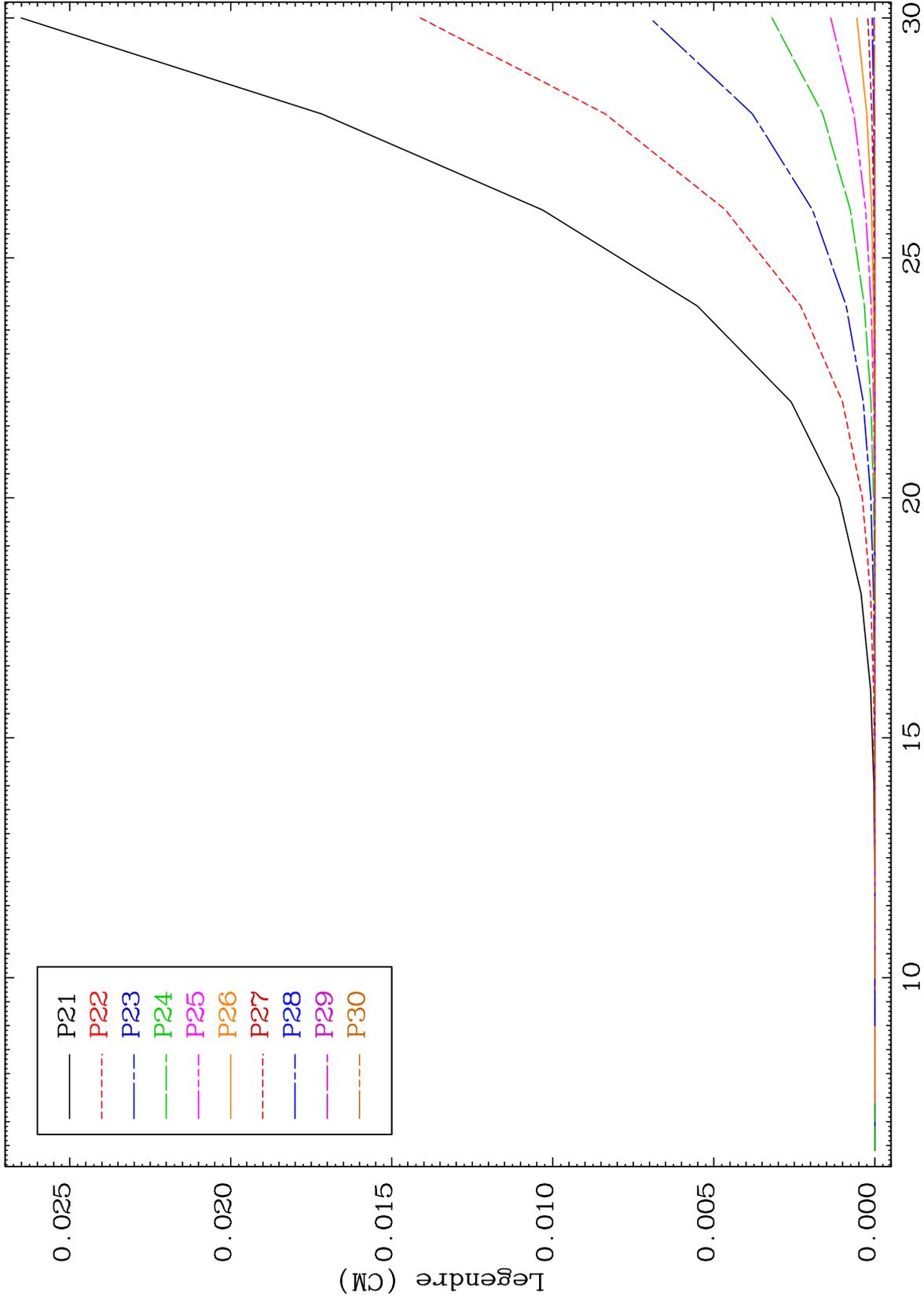
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Elastic Legendre Coefficients

98-Cf-256



18

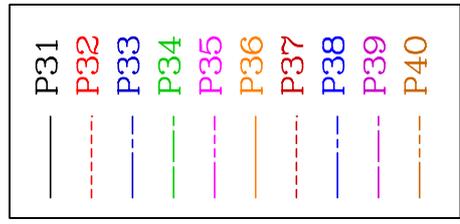
Incident Energy (MeV)

98-Cf-256

MAT 9873

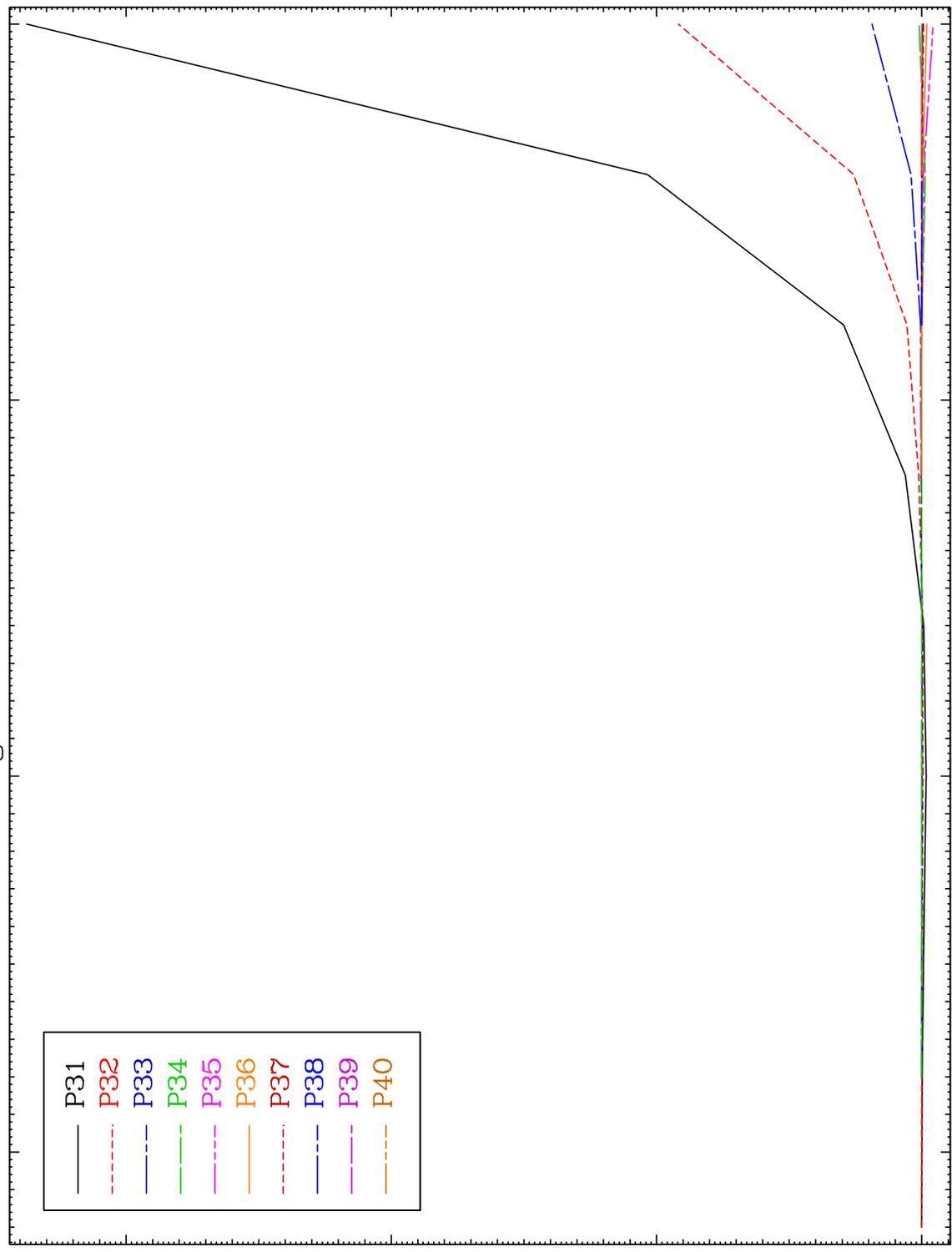
Elastic Legendre Coefficients

98-Cf-256



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

19

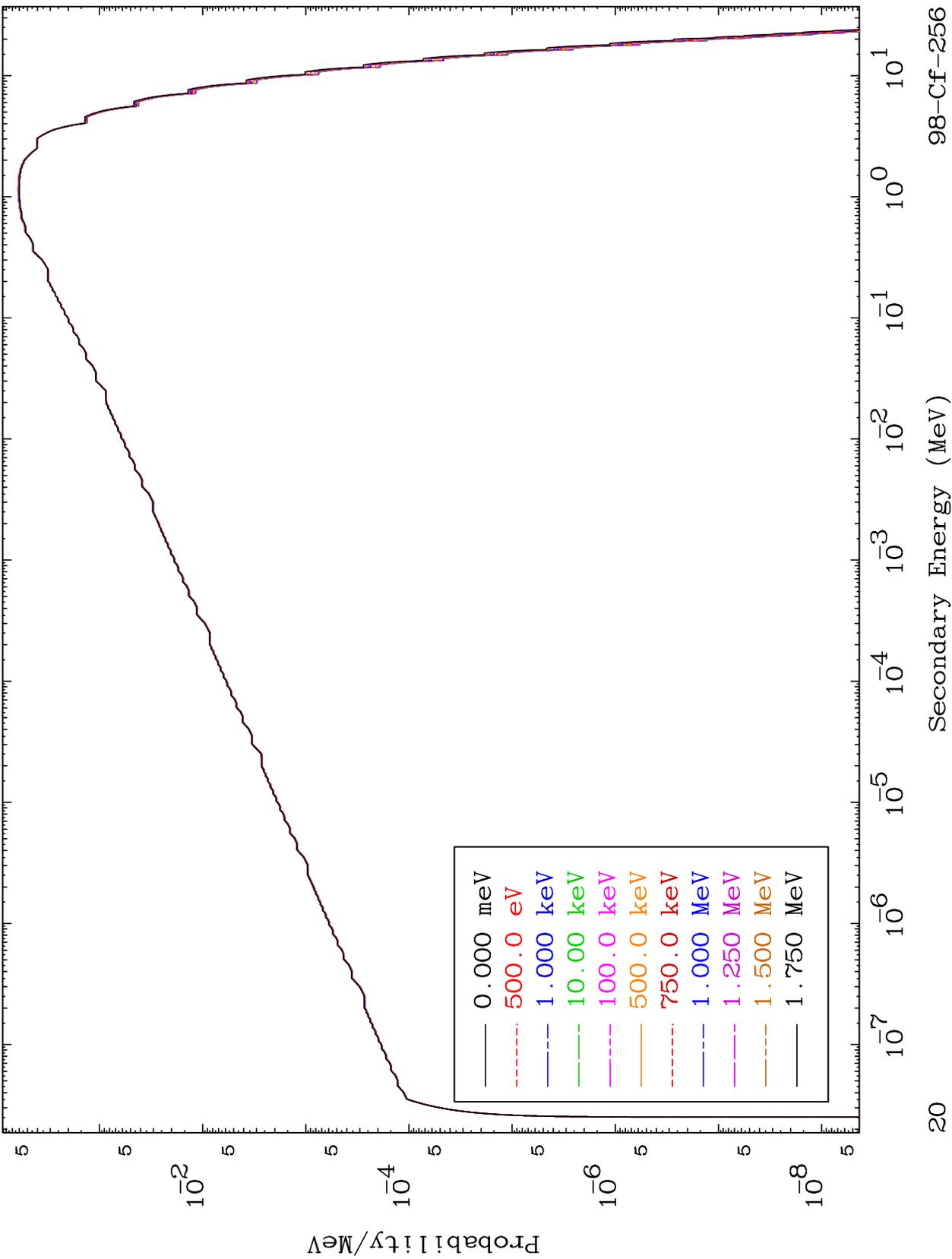
Incident Energy (MeV)

98-Cf-256

MAT 9873

Fission Energy Distribution

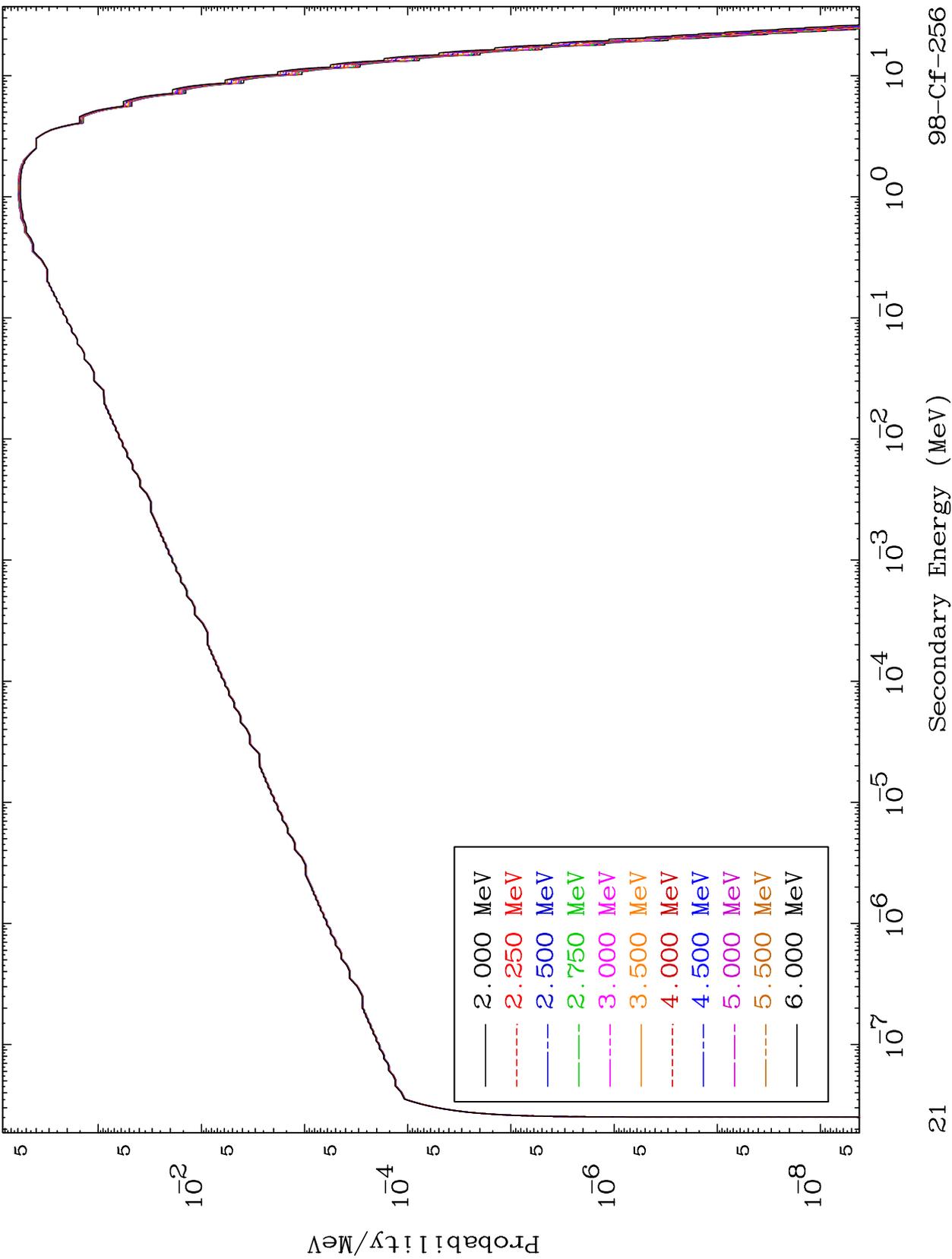
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MAT 9873

Fission Energy Distribution

98-Cf-256



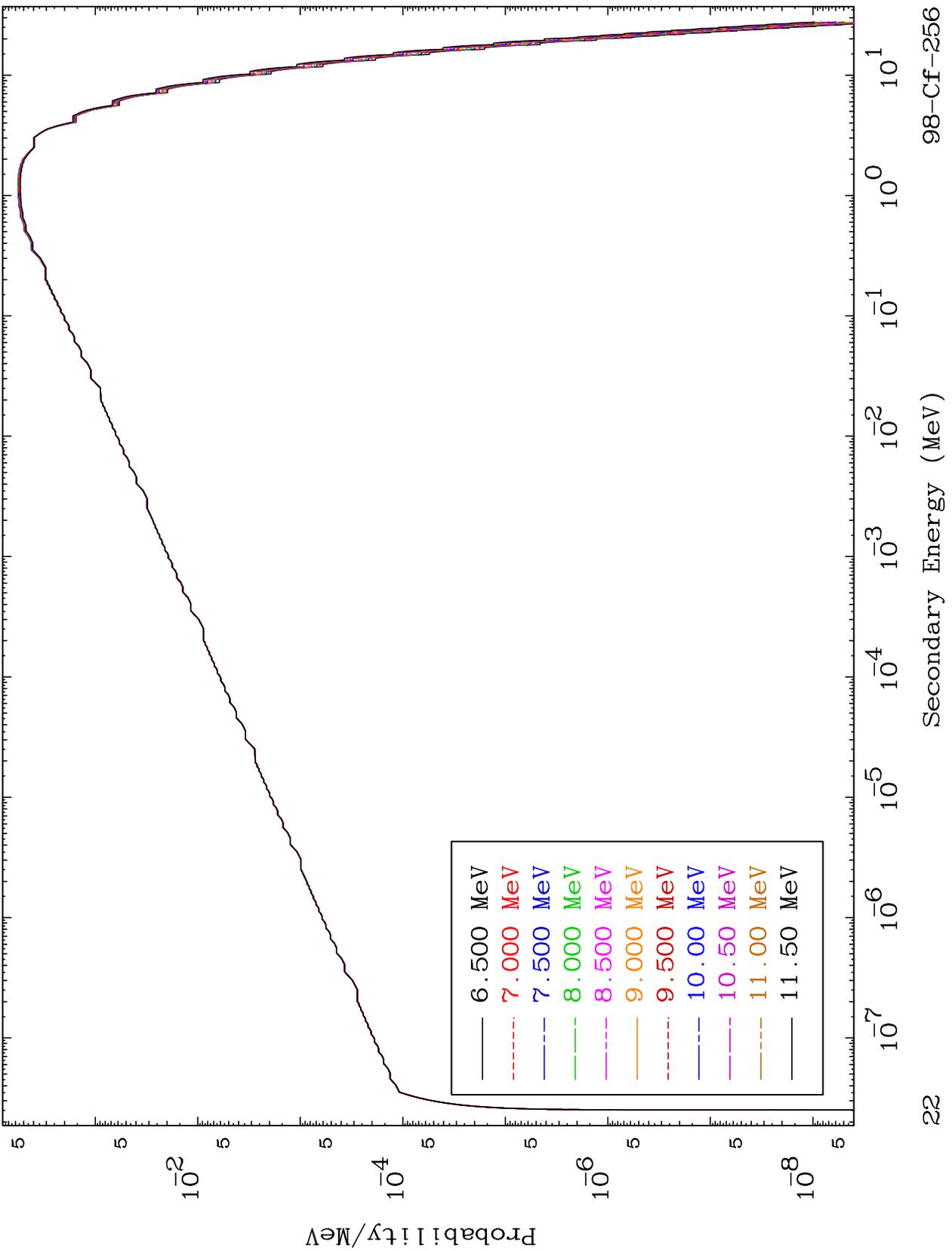
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Secondary Energy (MeV)

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Fission Energy Distribution

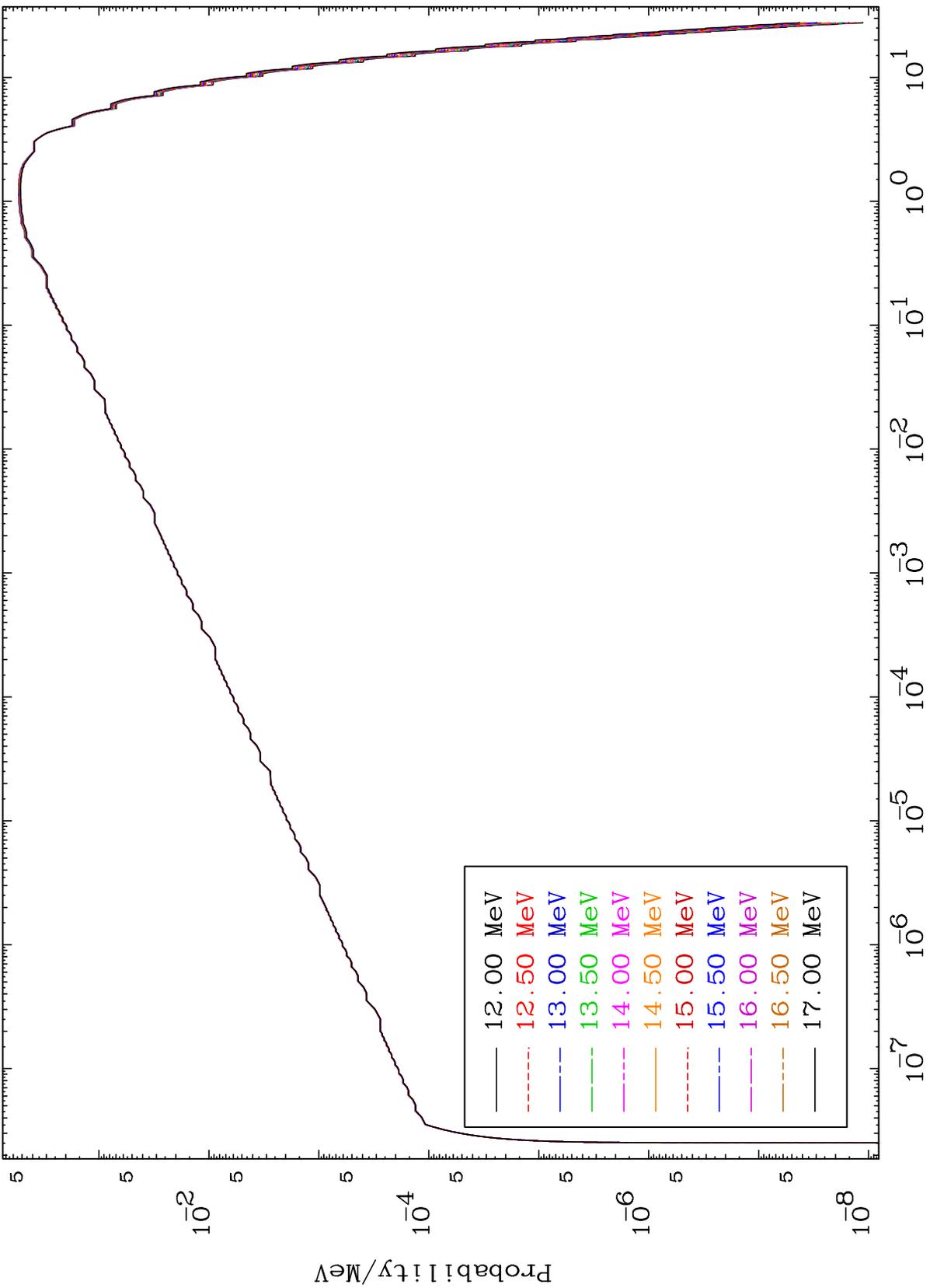
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Fission
Energy Distribution

98-Cf-256



98-Cf-256

MAT 9873

Fission Energy Distribution

98-Cf-256

