

Program EVALPLOT
(Version 2017-1)

by

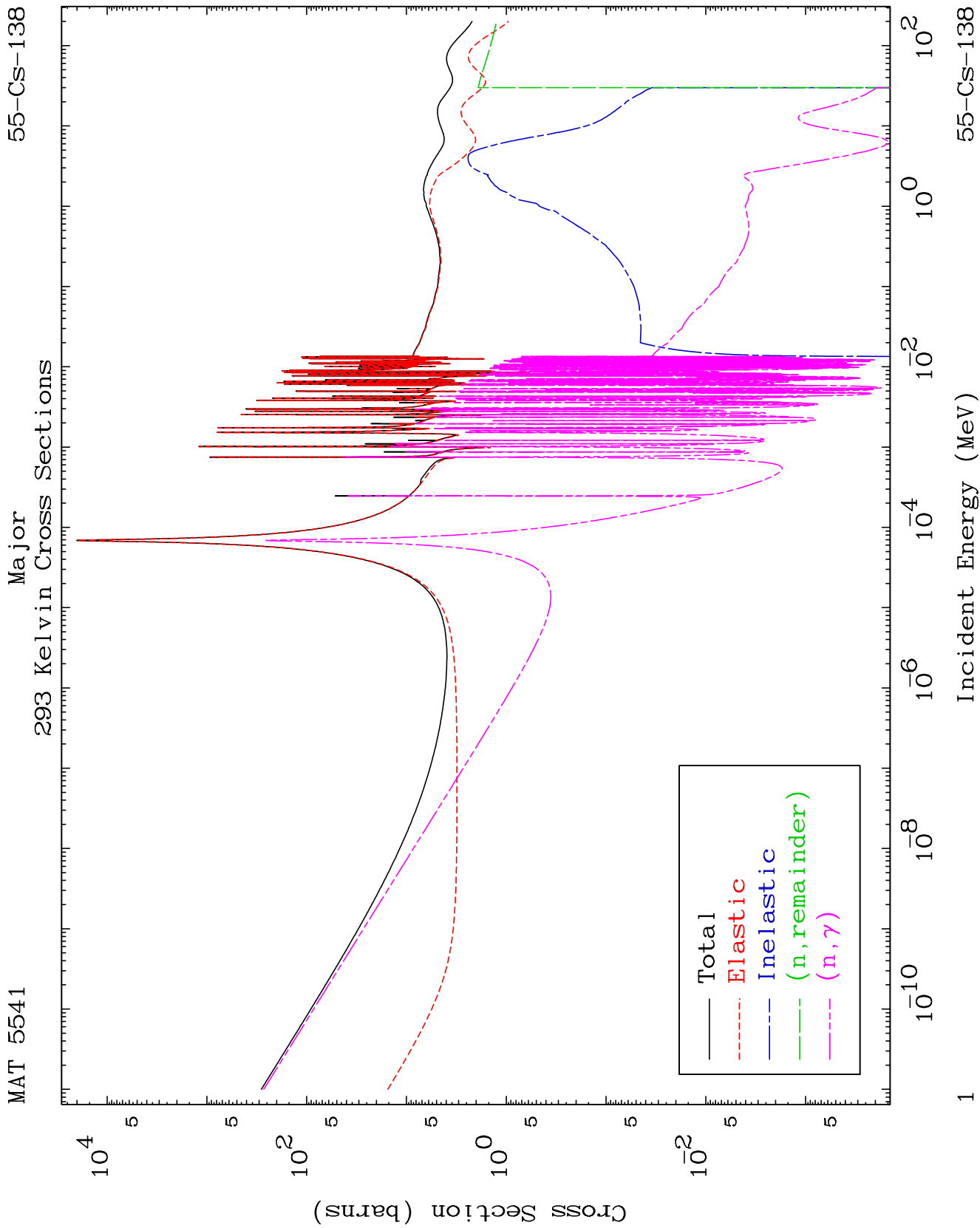
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(Present Contact Information)

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Press Mouse Button to Start

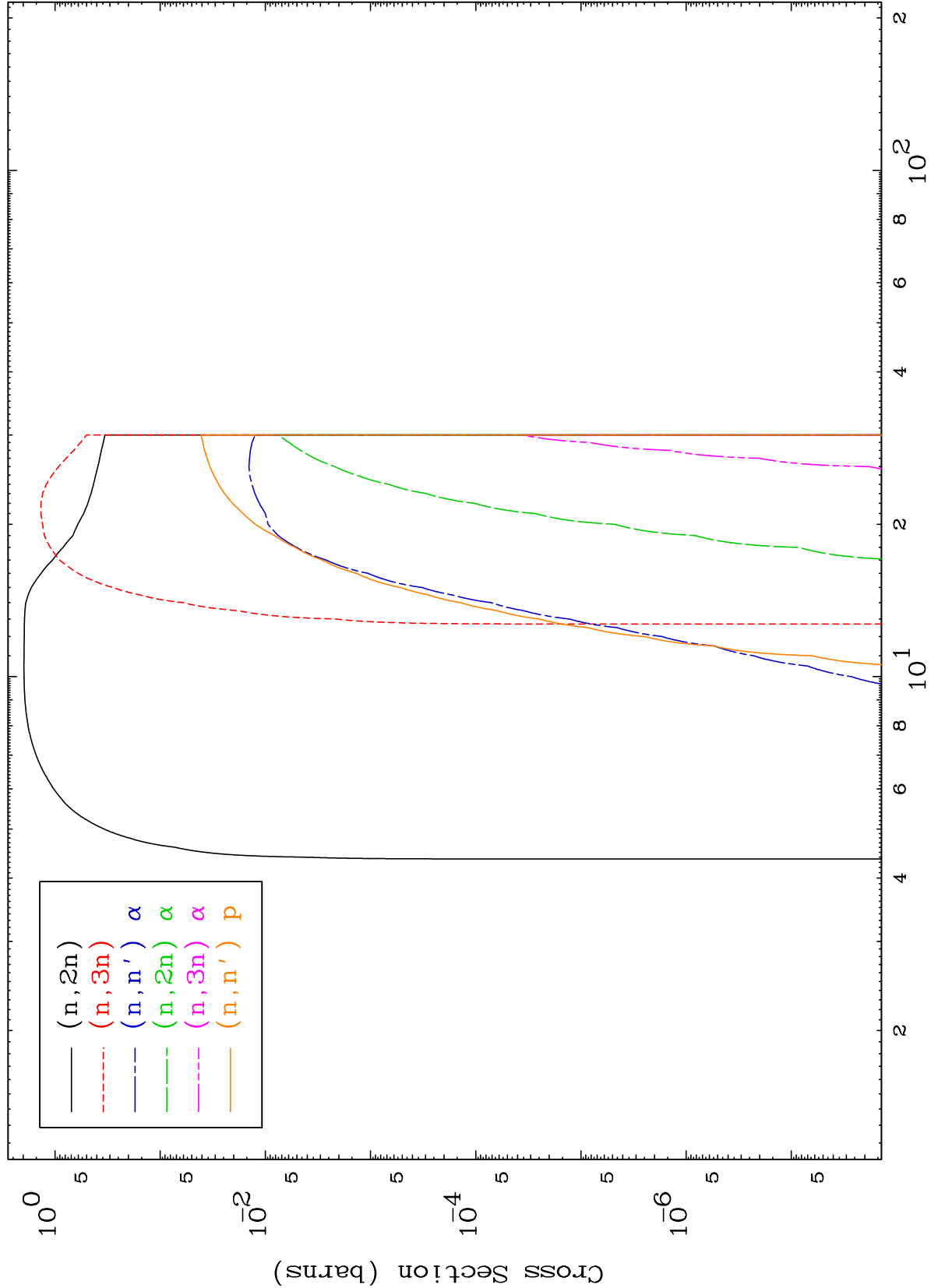


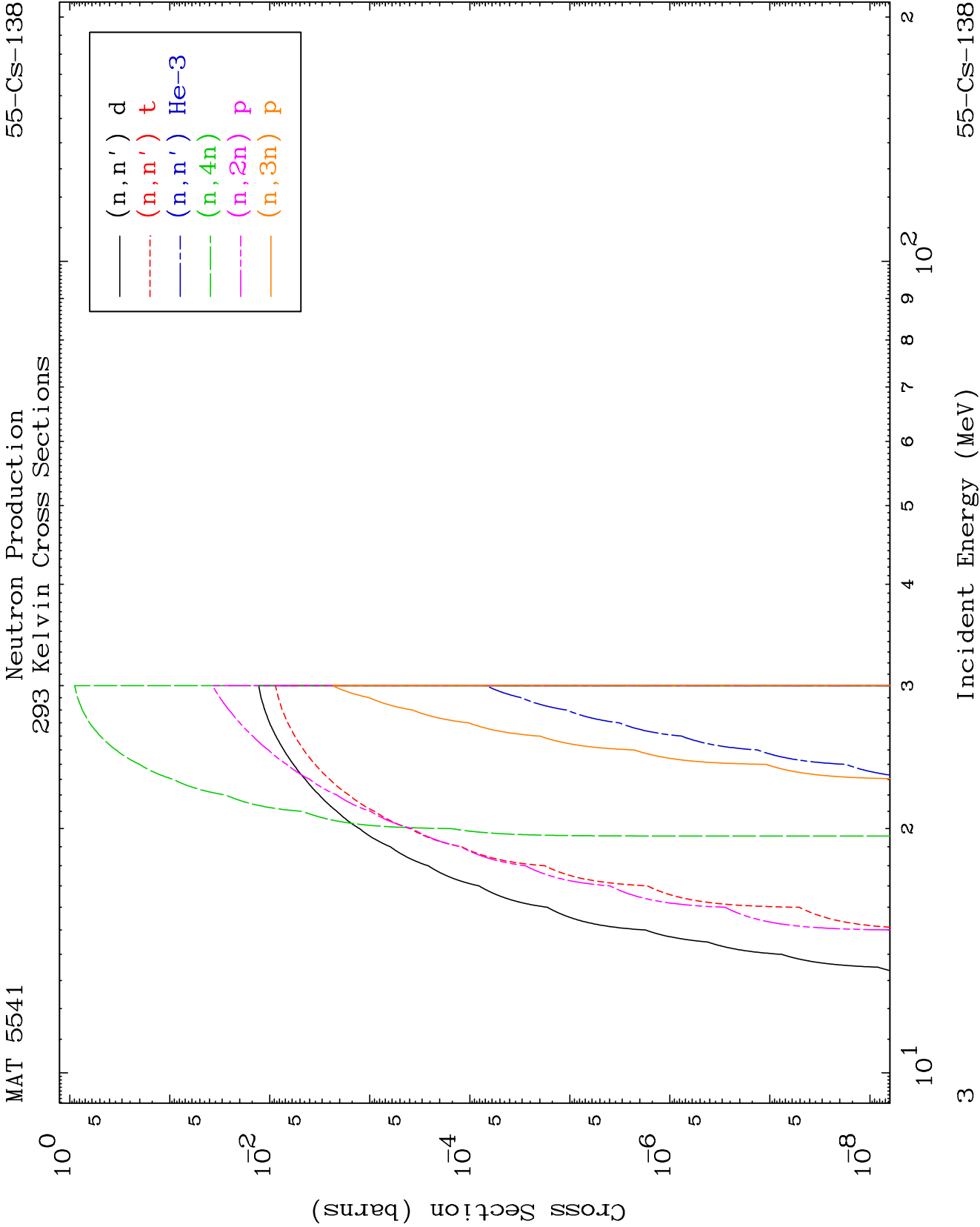
MAT 5541

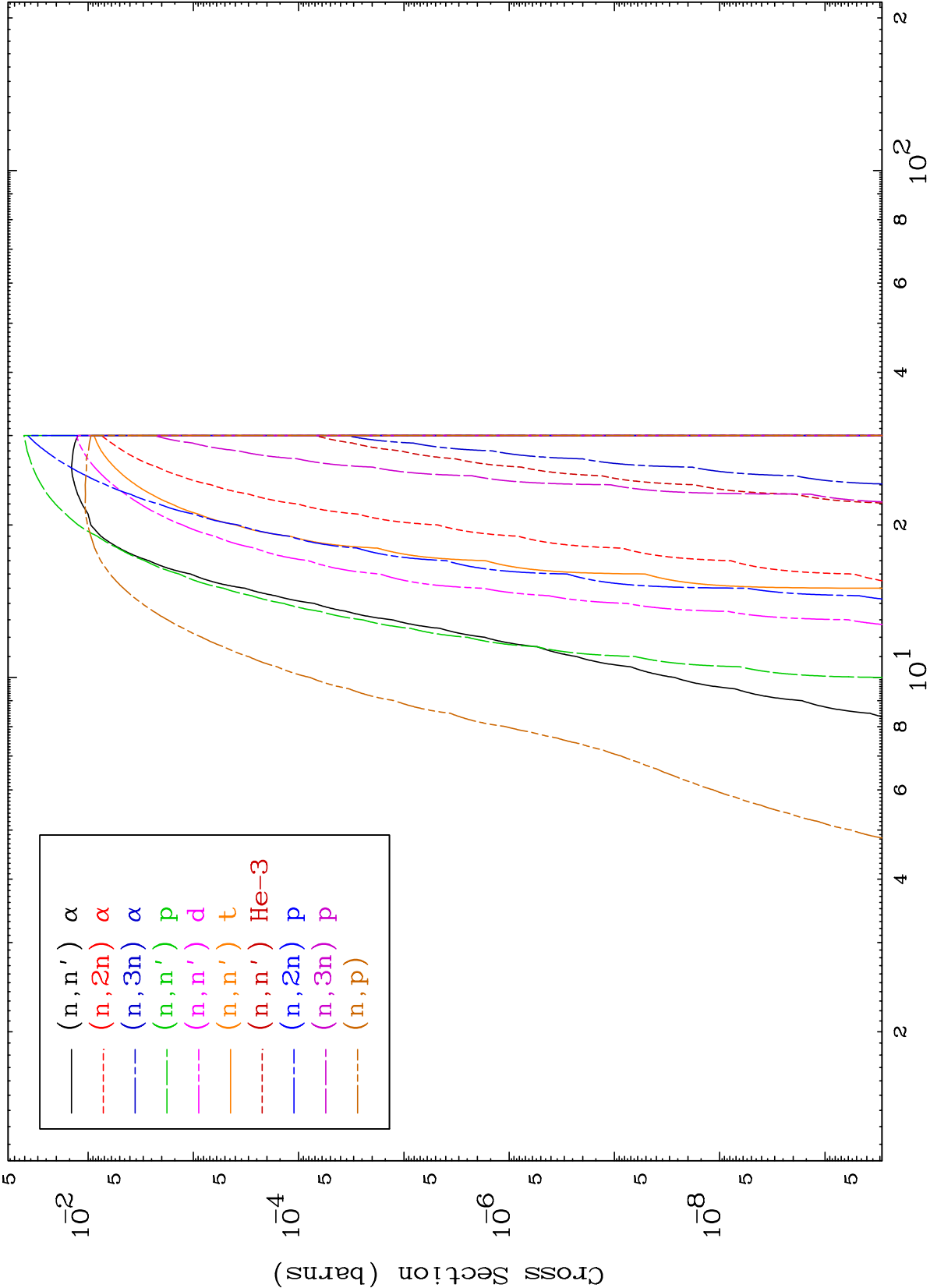
Neutron Production

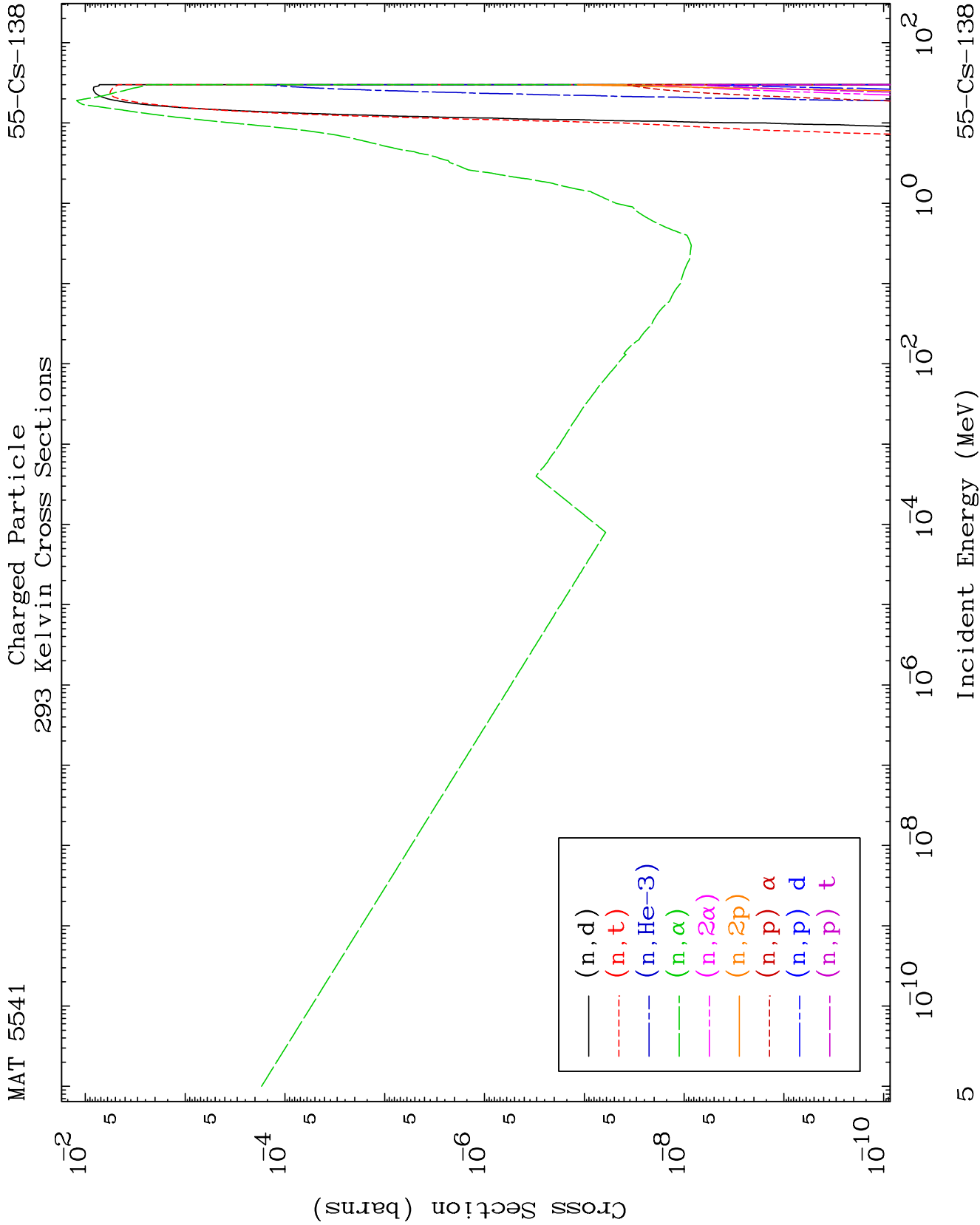
55-Cs-138

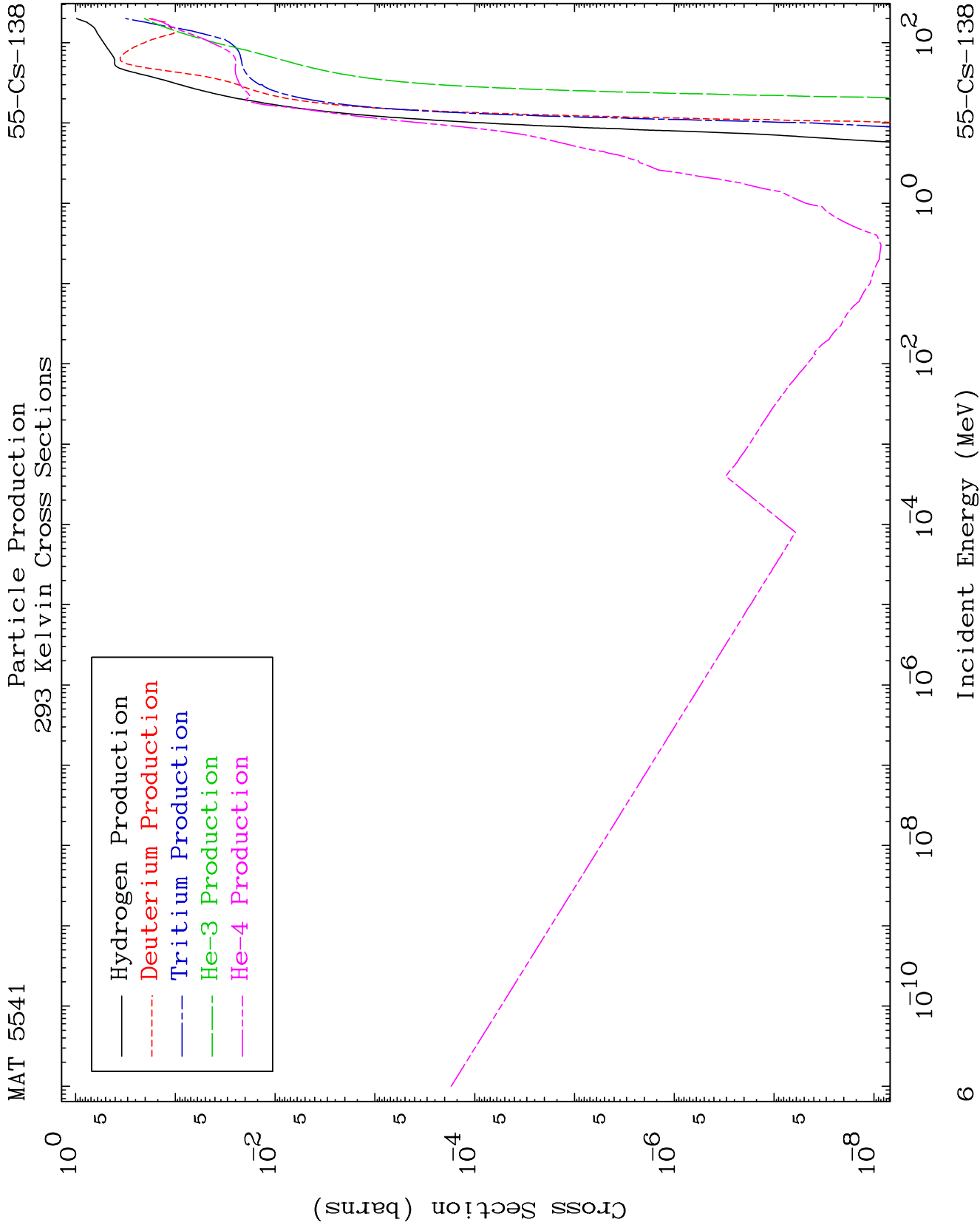
293 Kelvin Cross Sections







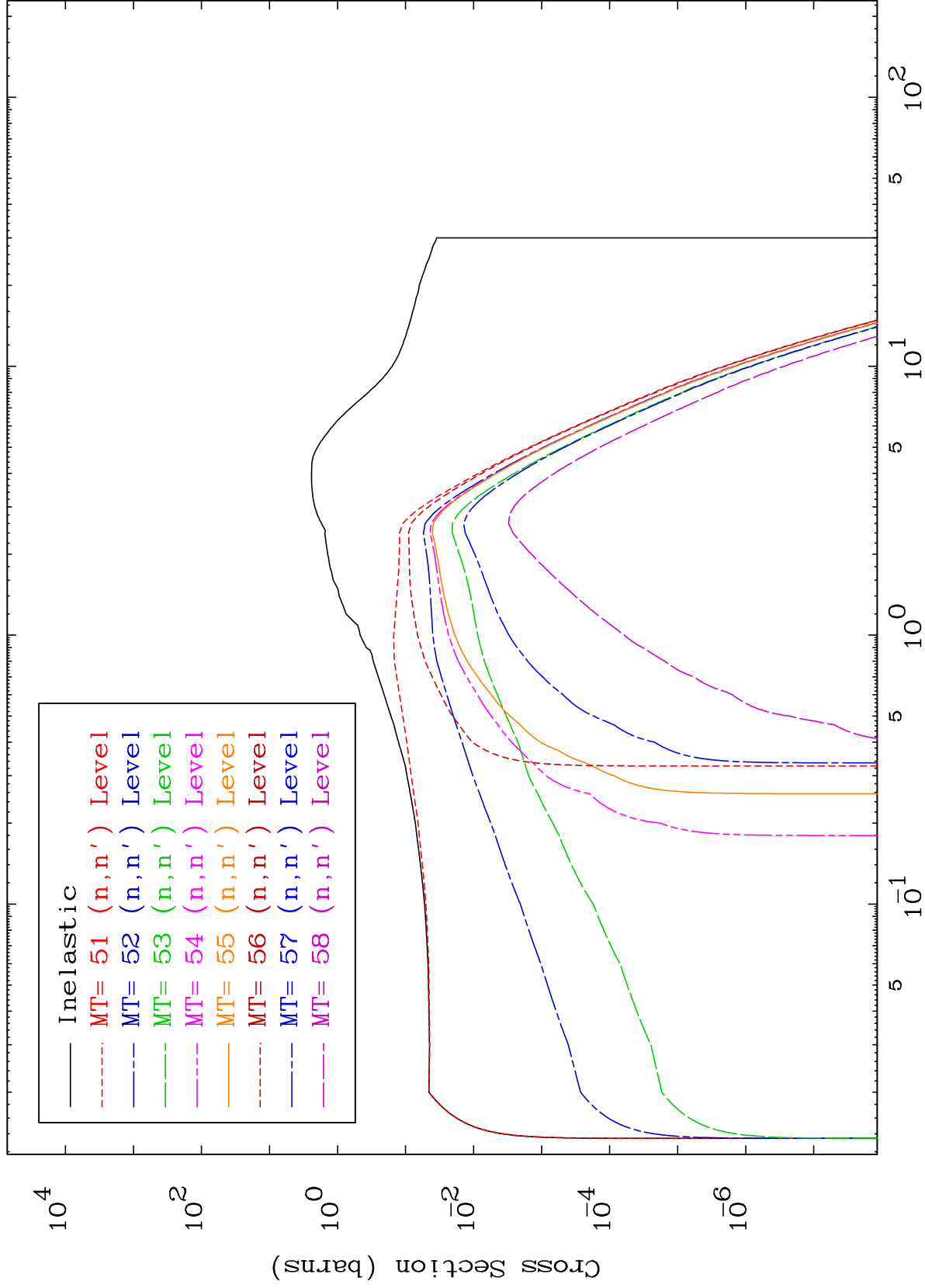


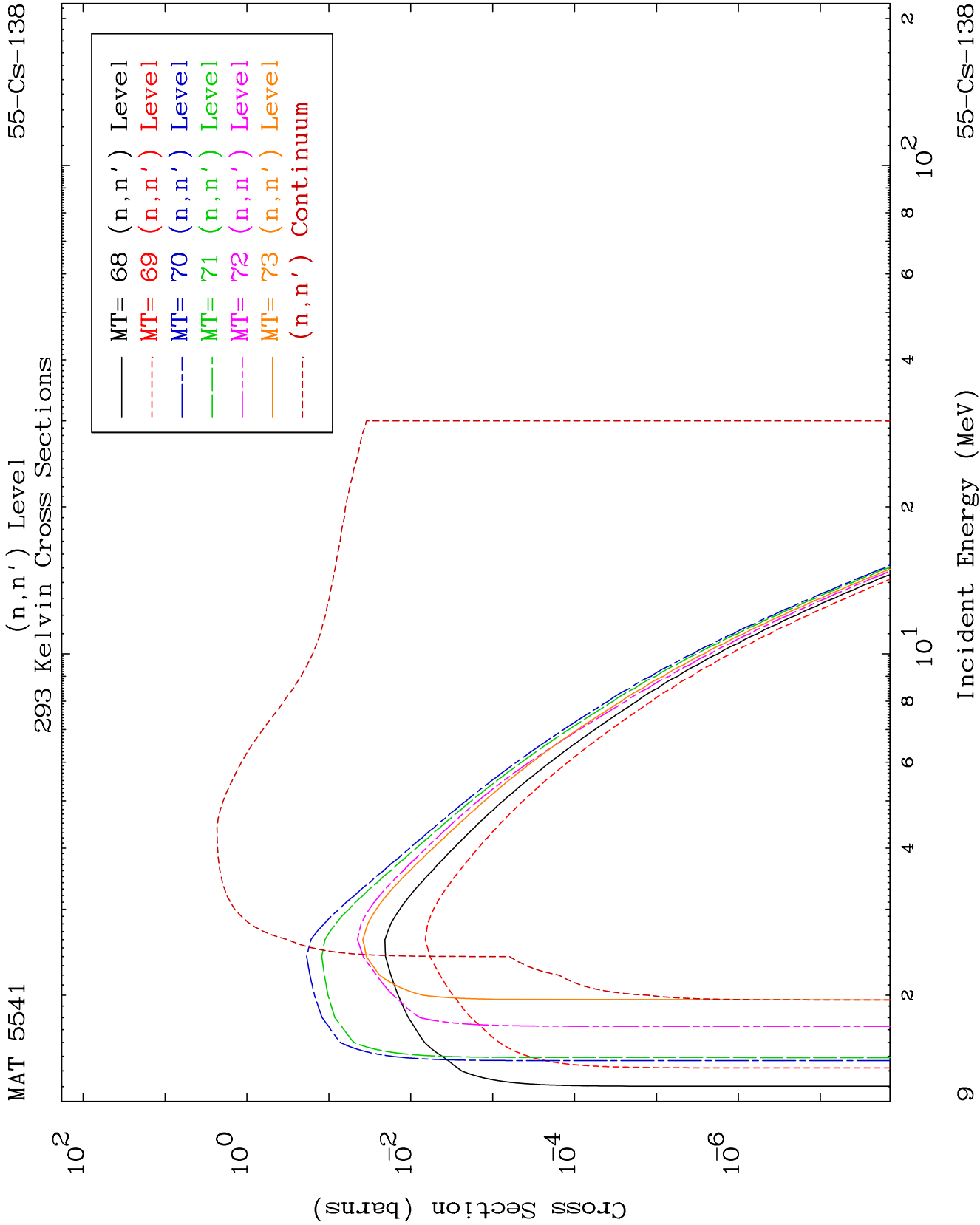


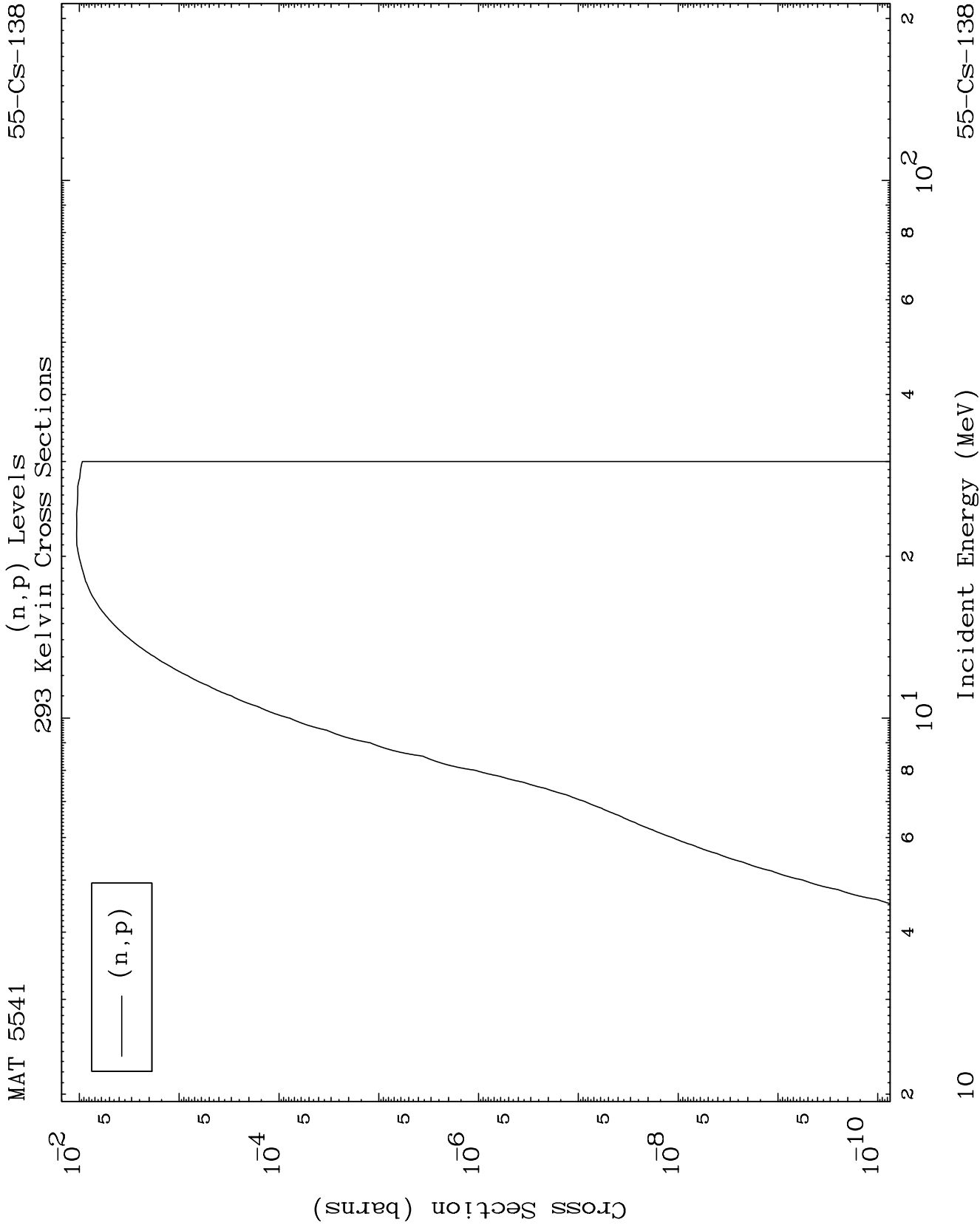
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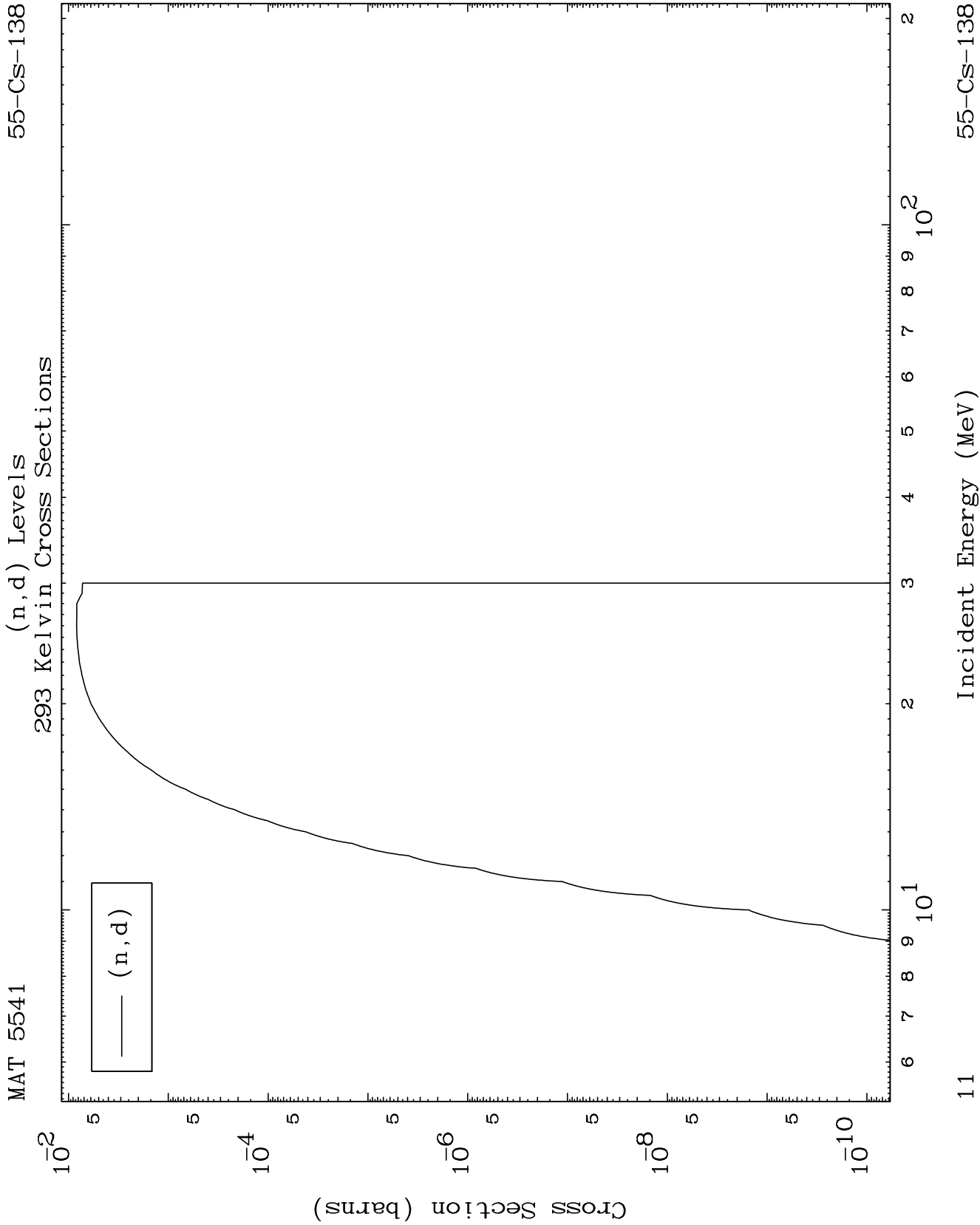
(n,n') Level
293 Kelvin Cross Sections

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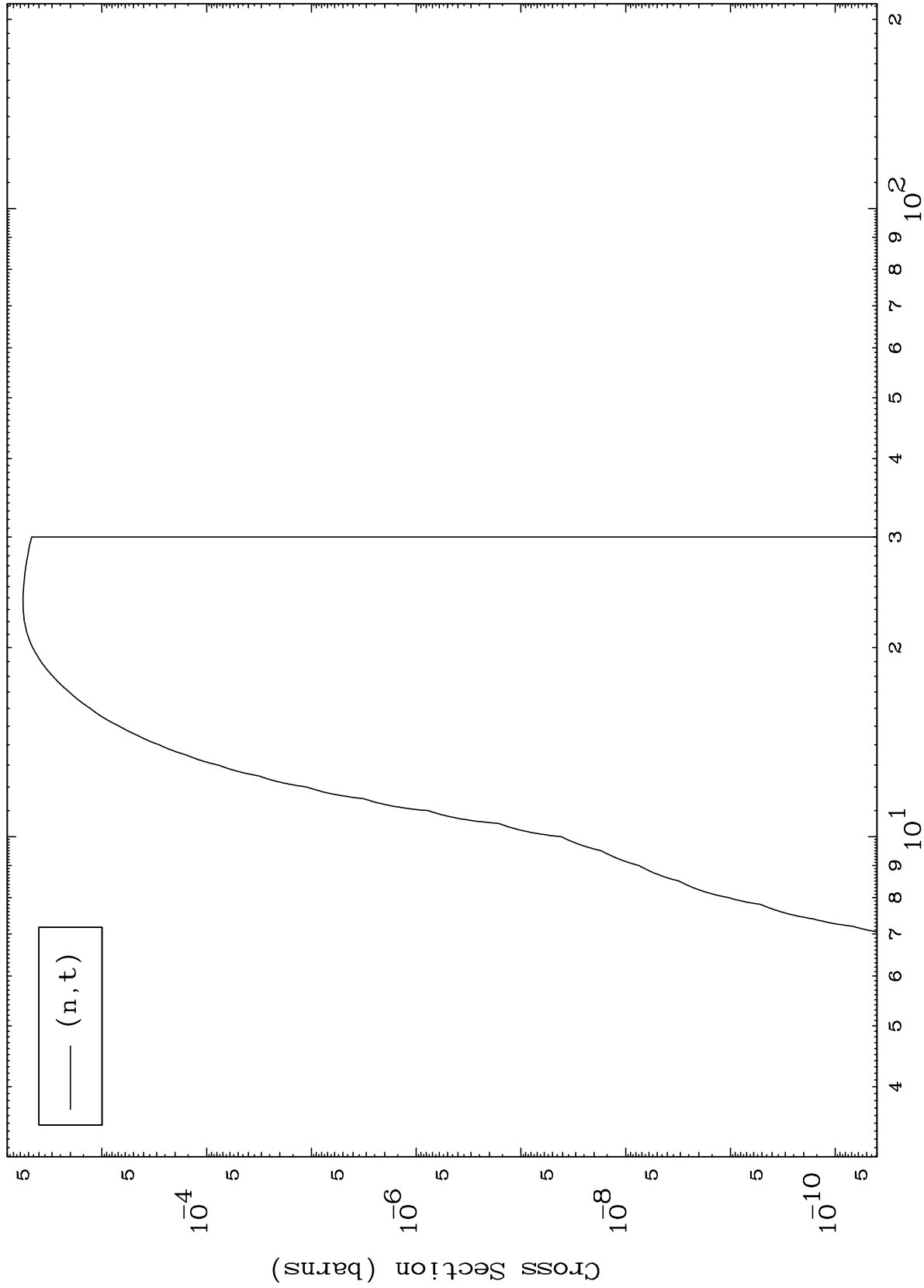




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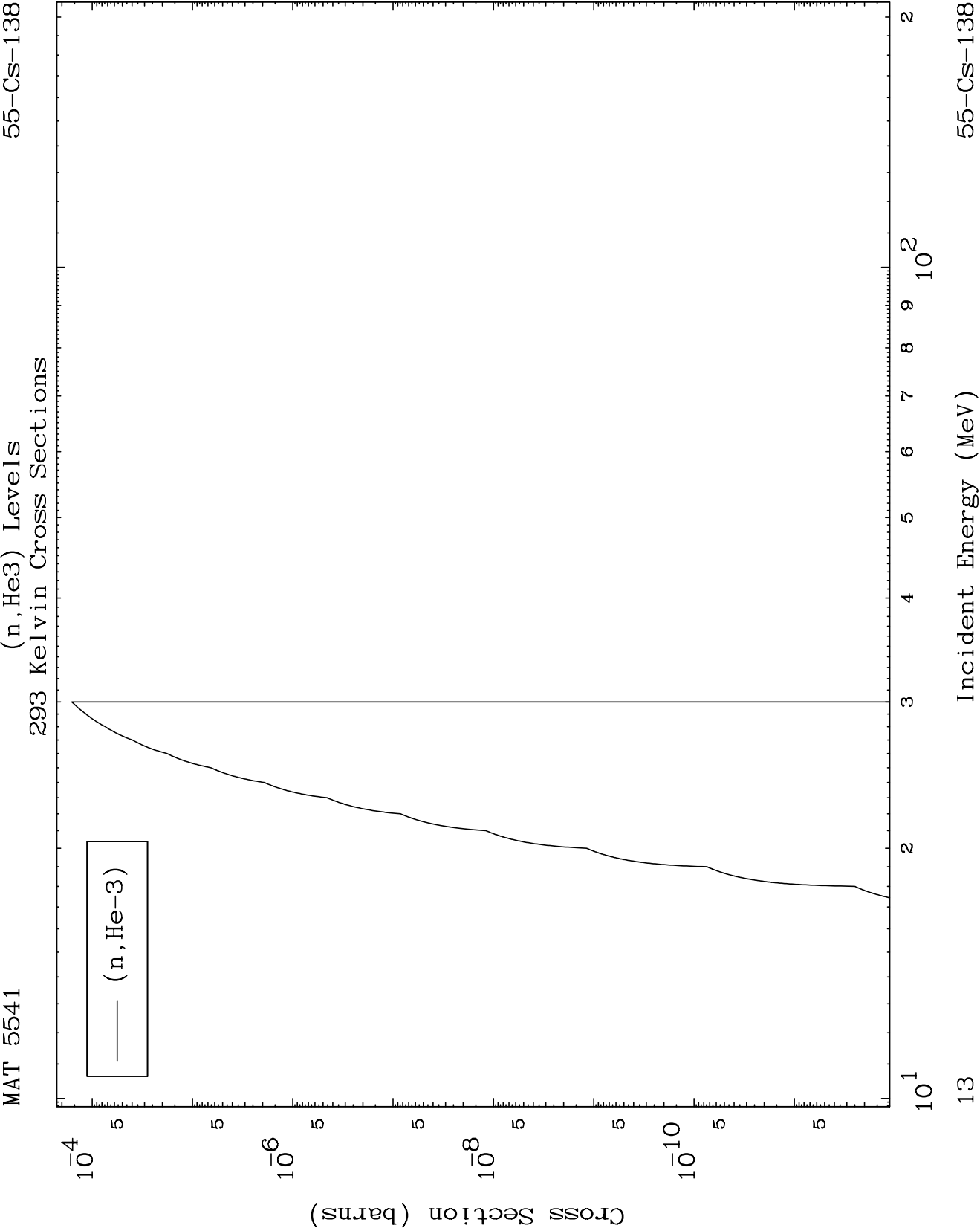
(n,t) Levels
293 Kelvin Cross Sections

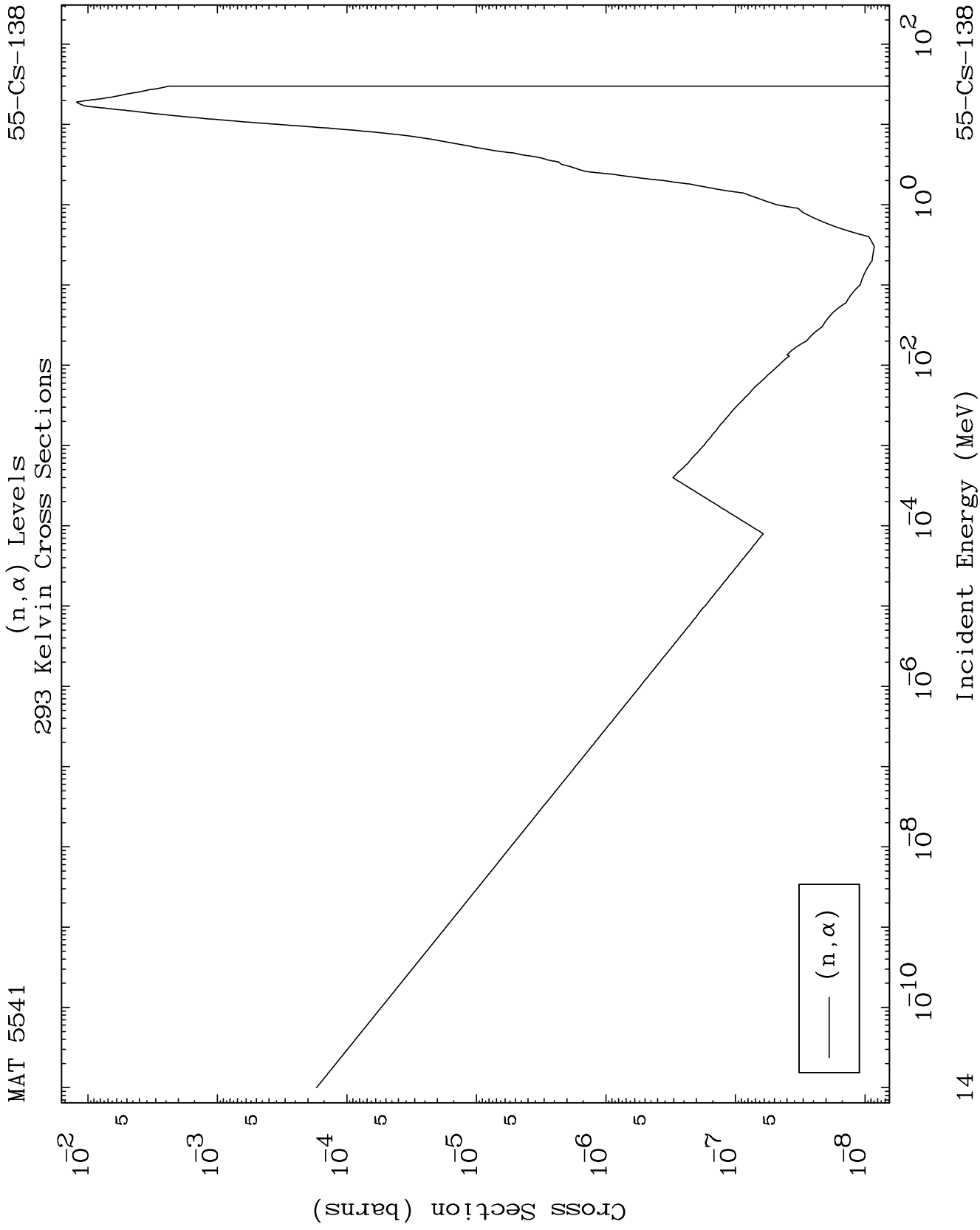


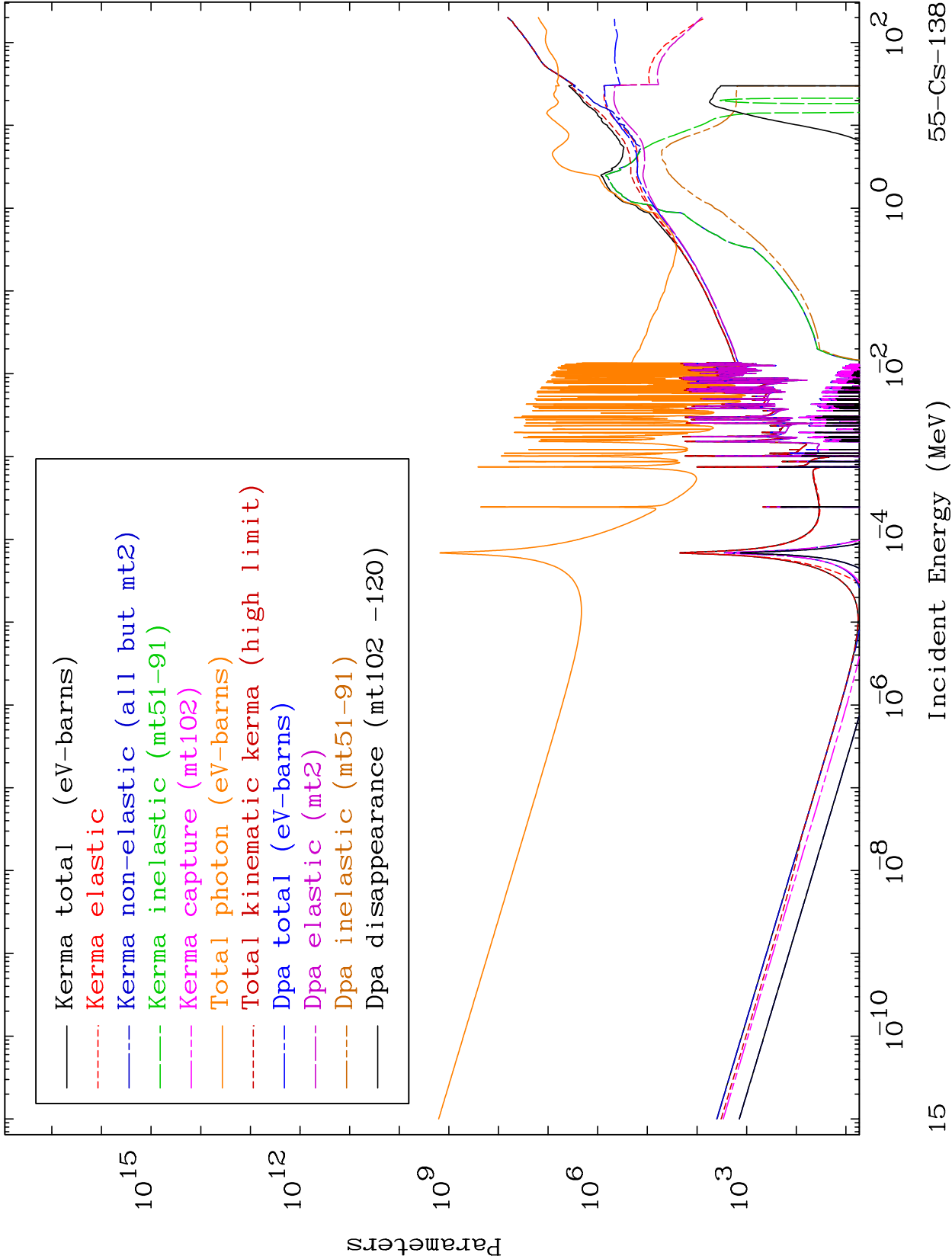
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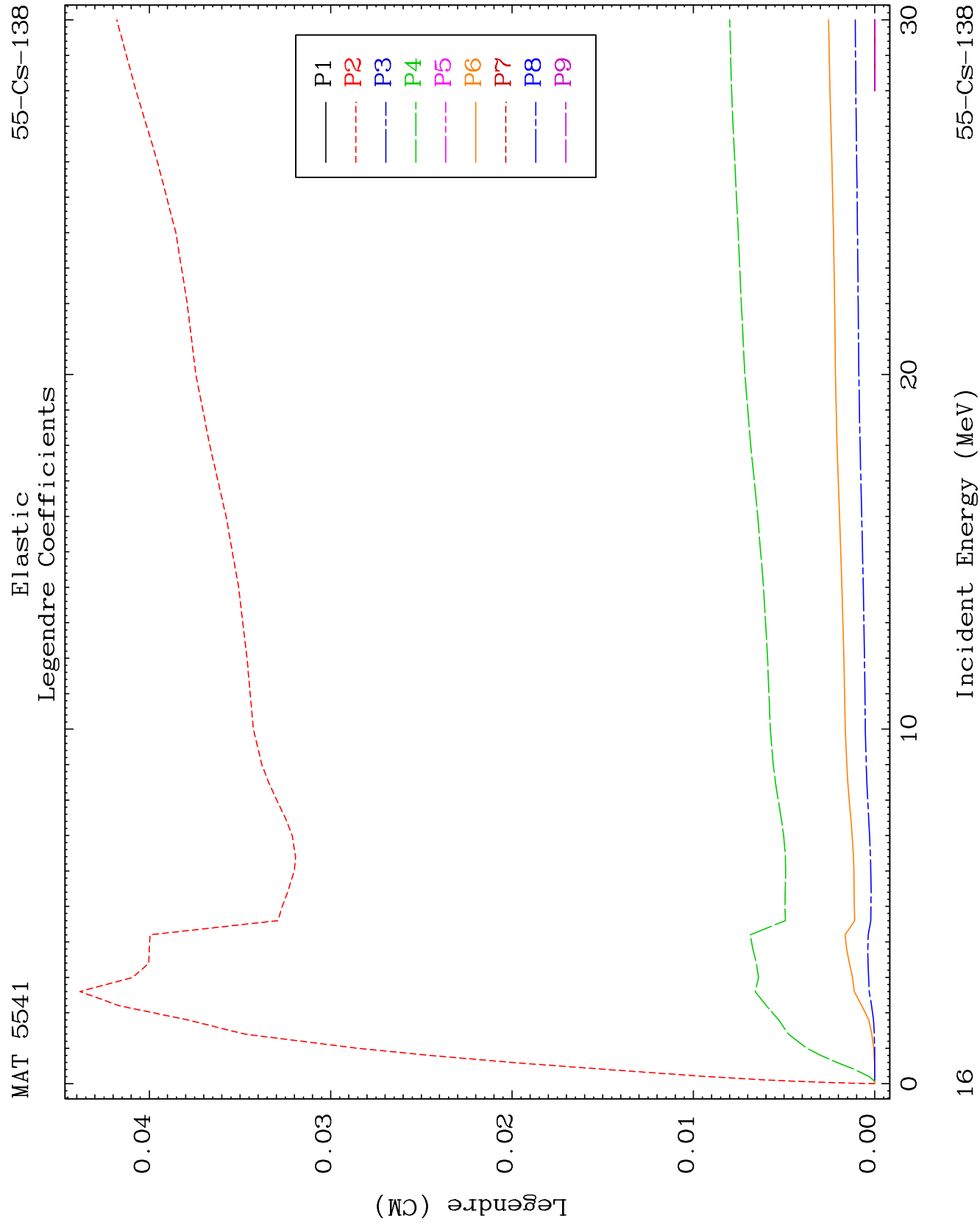
Incident Energy (MeV)

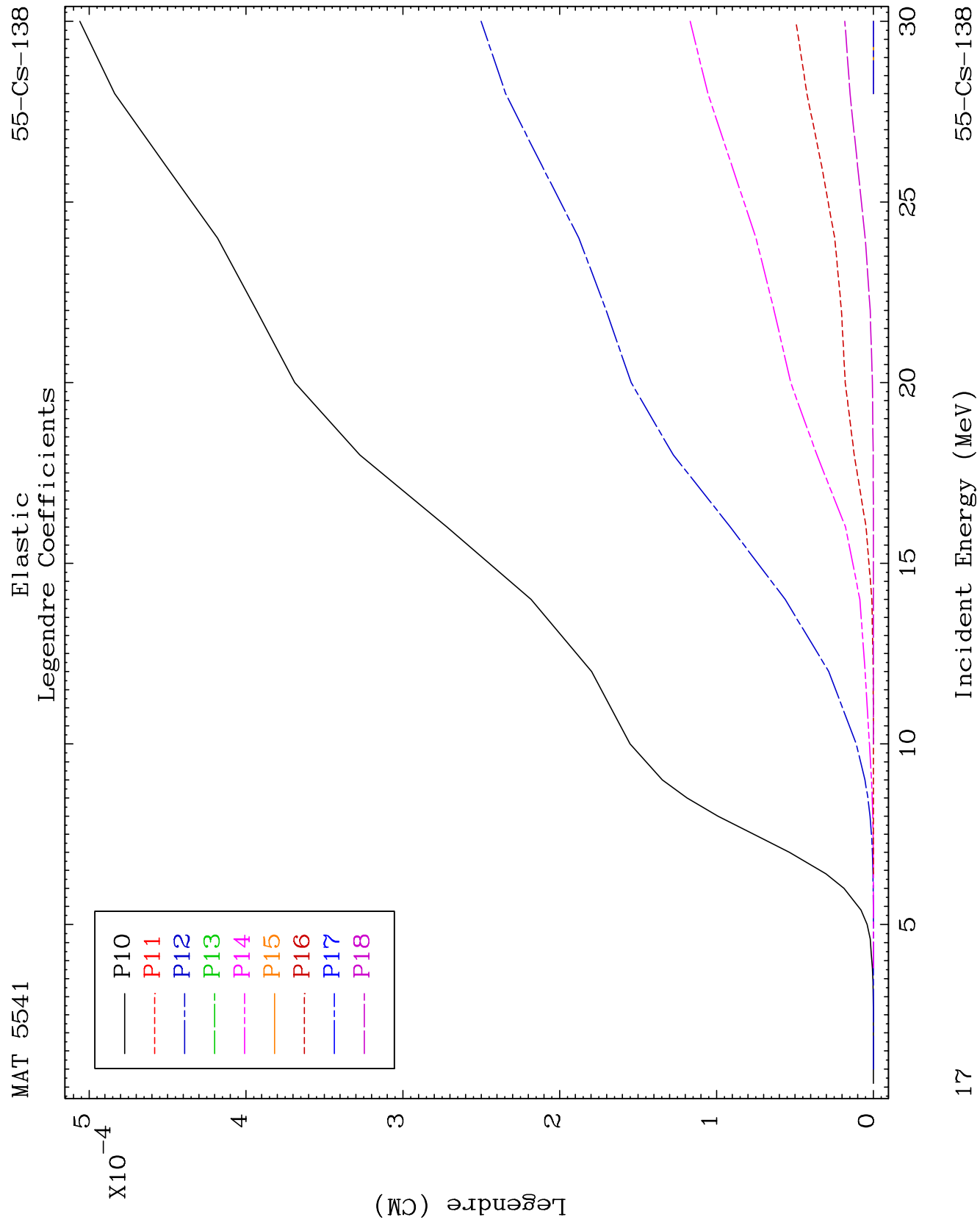
55-Cs-138

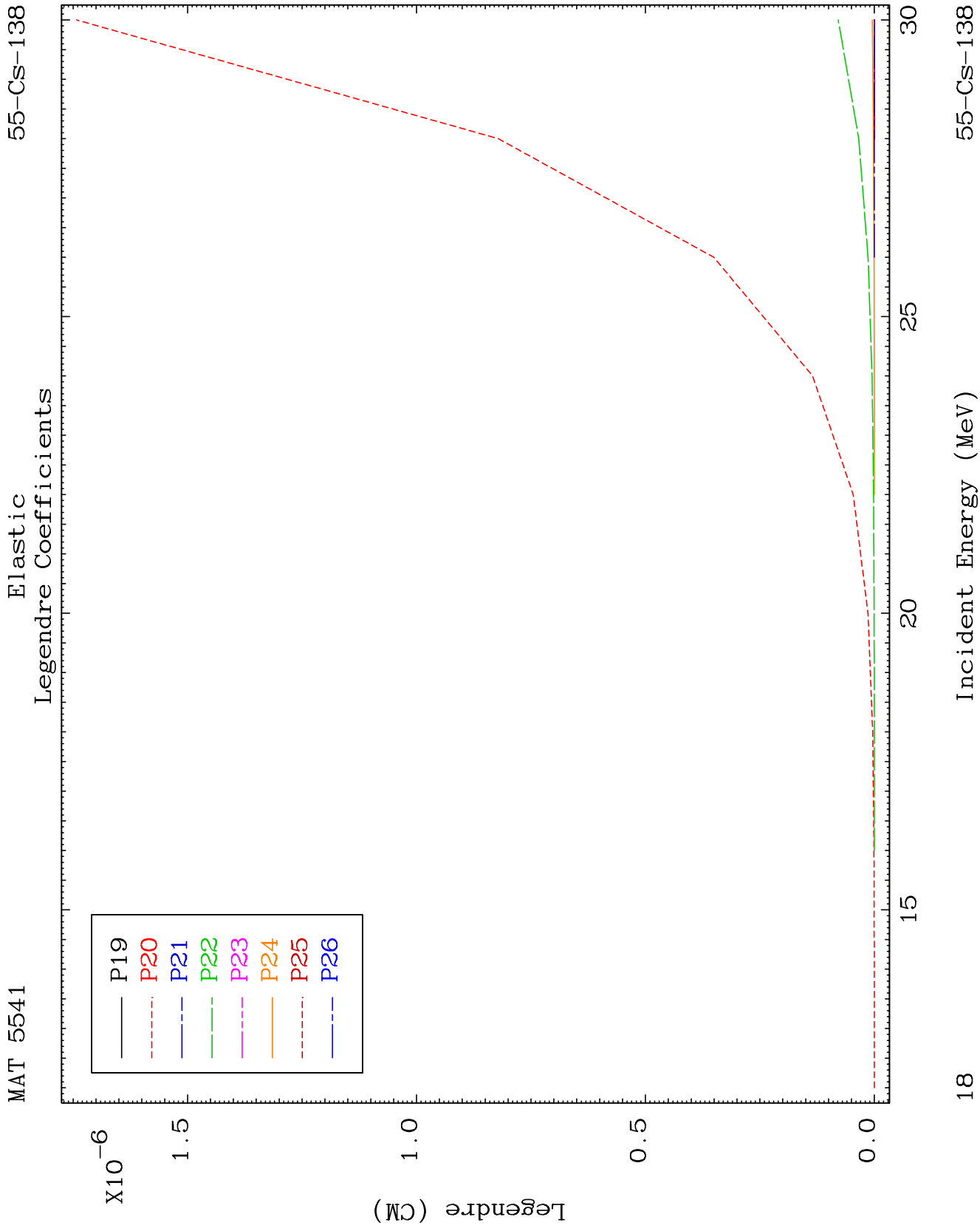








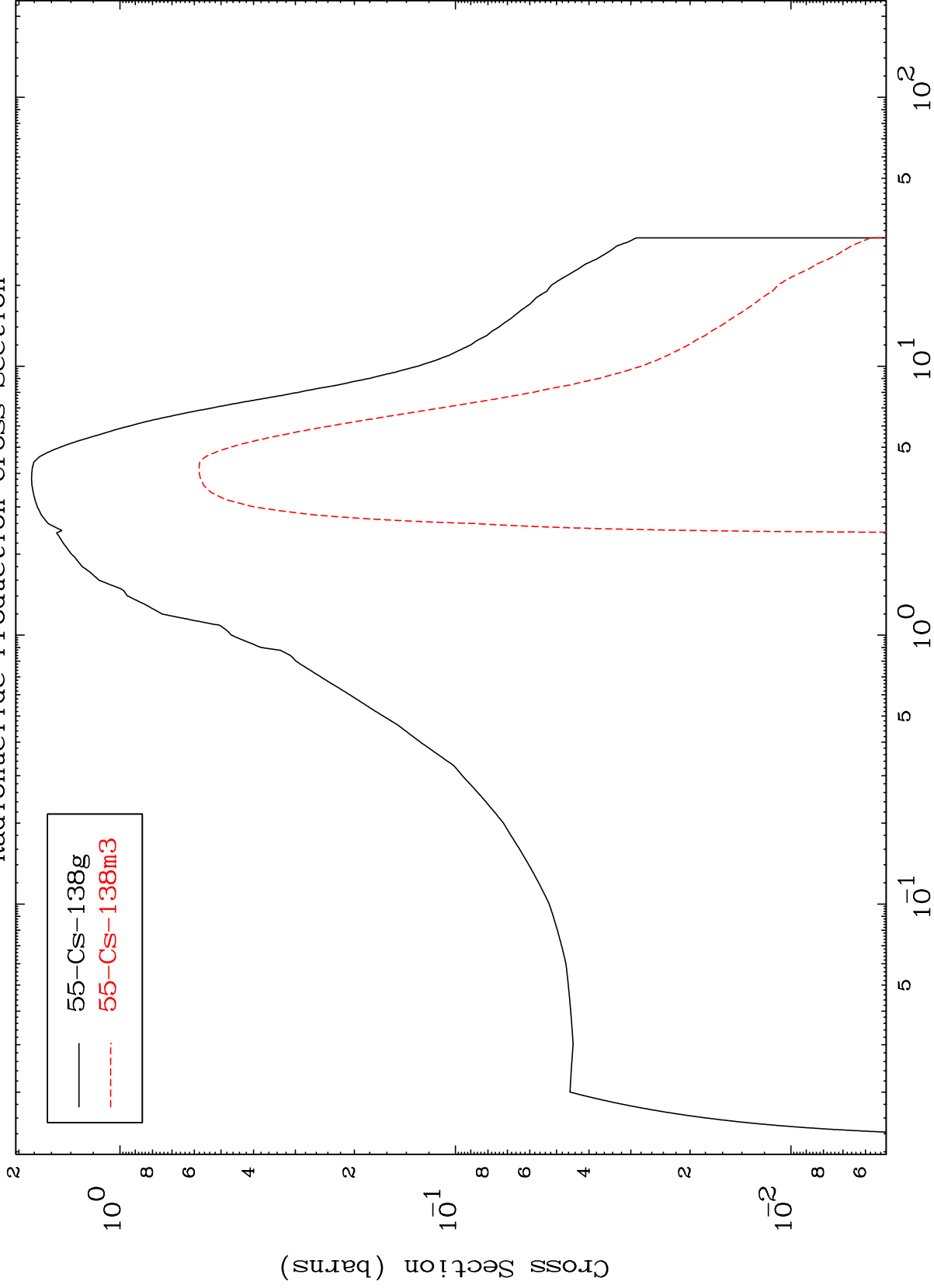




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Radionuclide Production Cross Section



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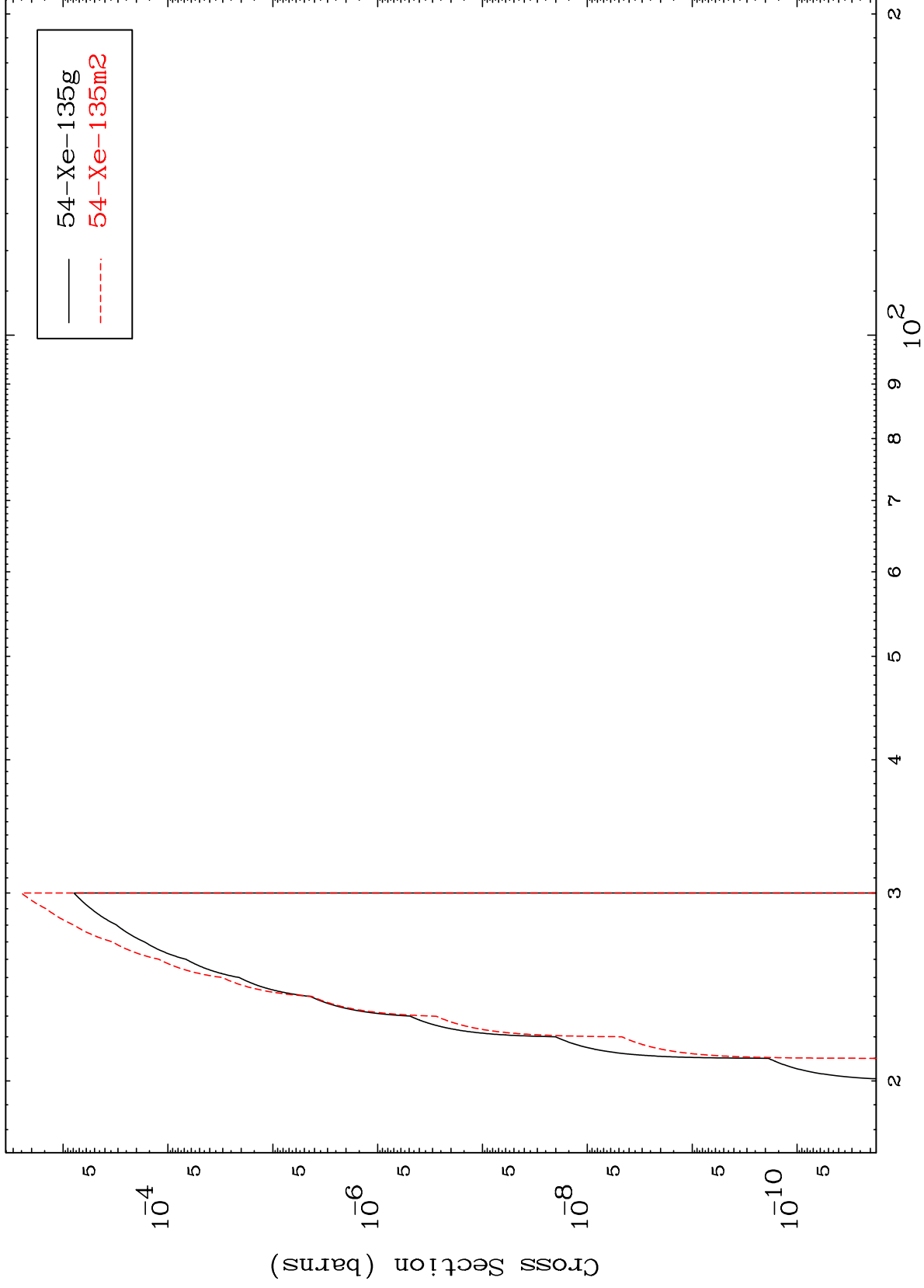
Incident Energy (MeV)

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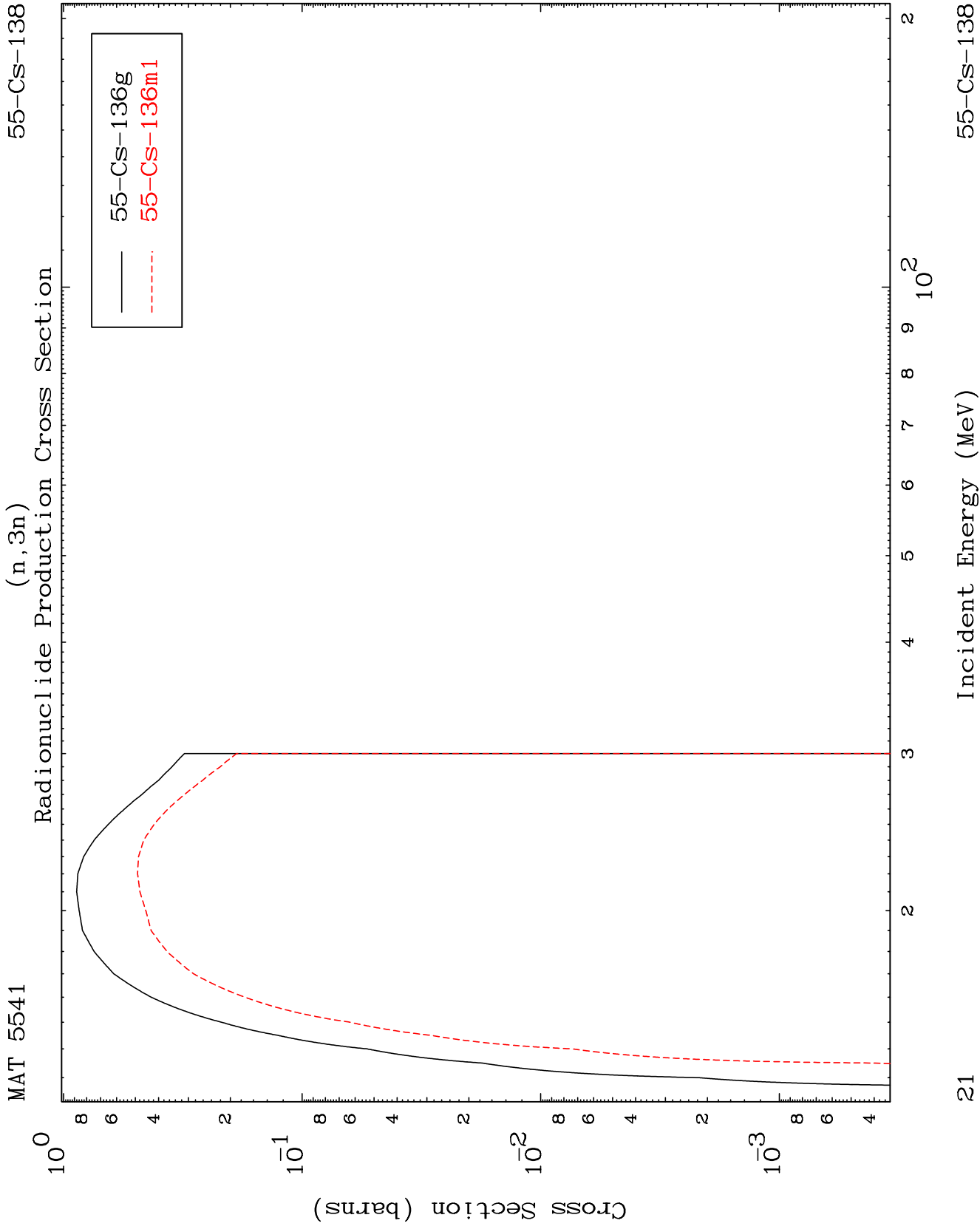
(n,2n) d
Radionuclide Production Cross Section

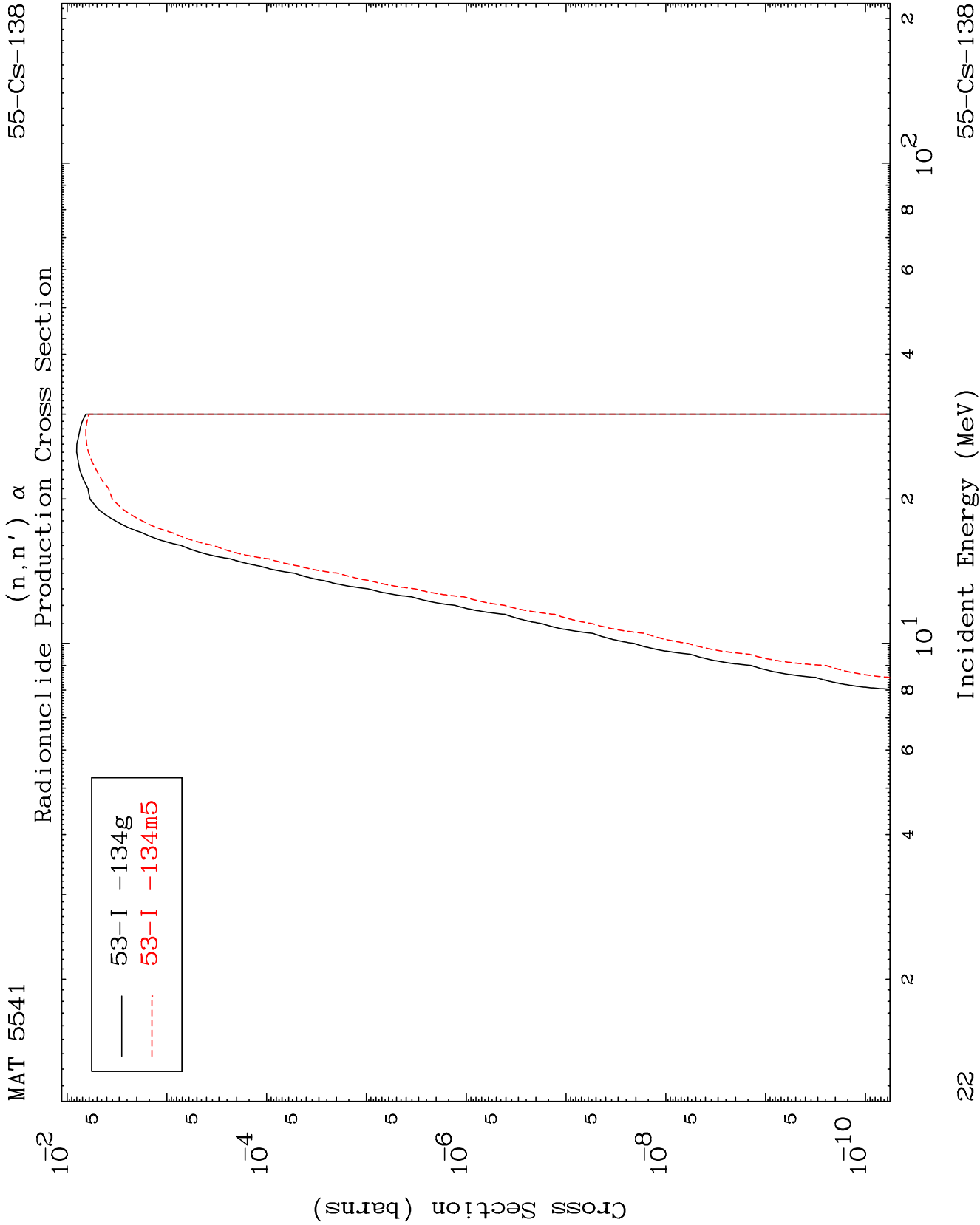


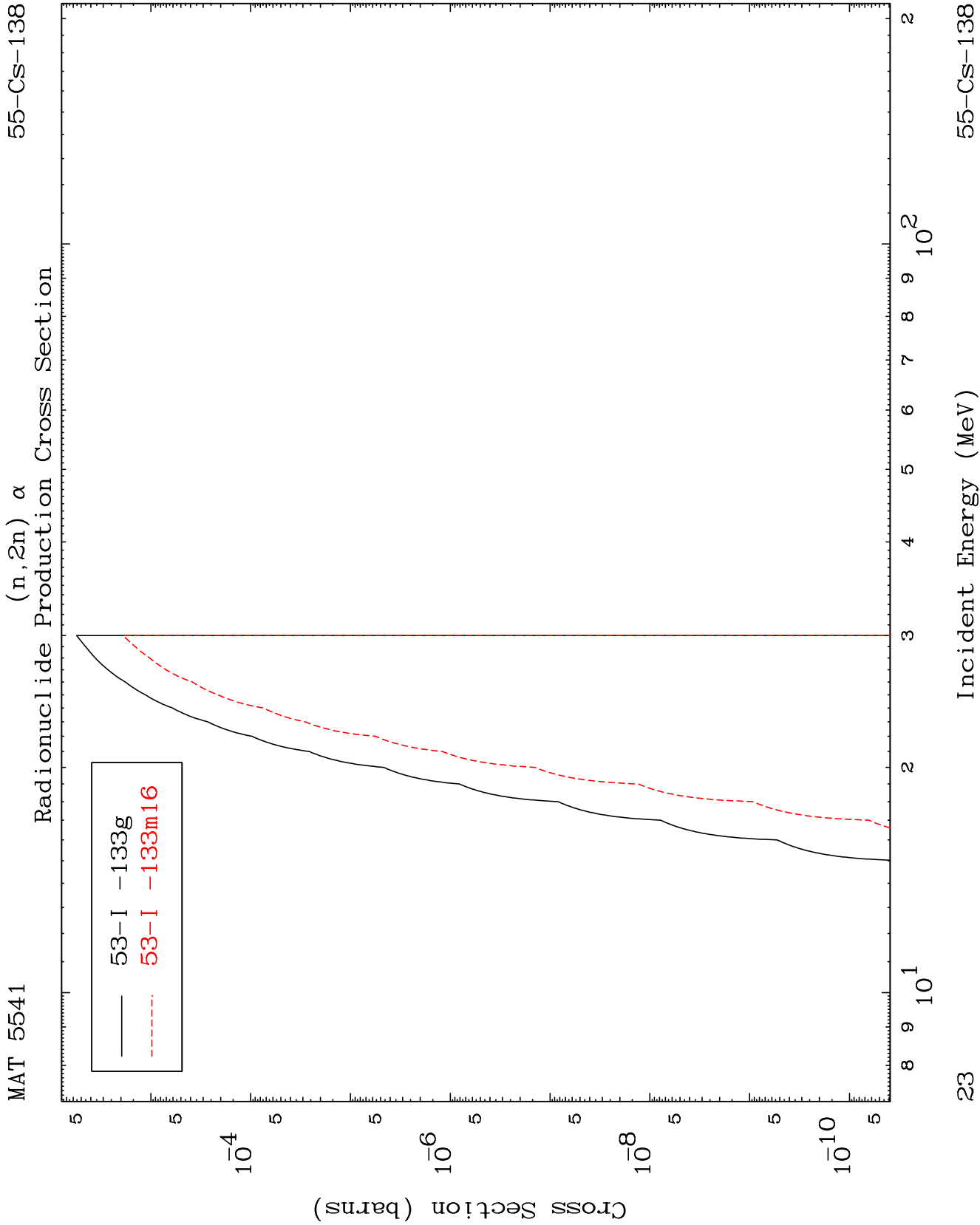
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Incident Energy (MeV)

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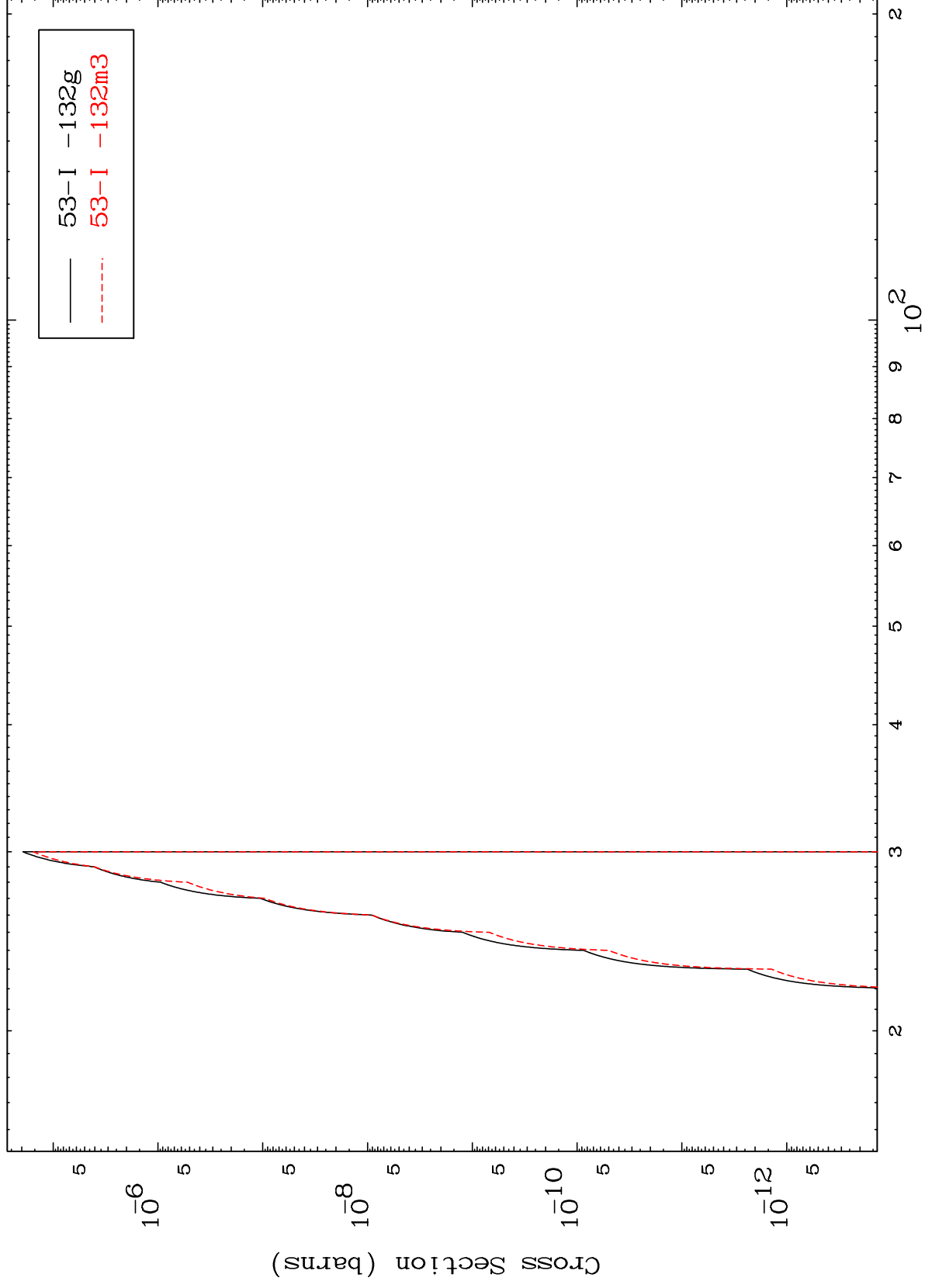




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$(n,3n) \alpha$
Radionuclide Production Cross Section

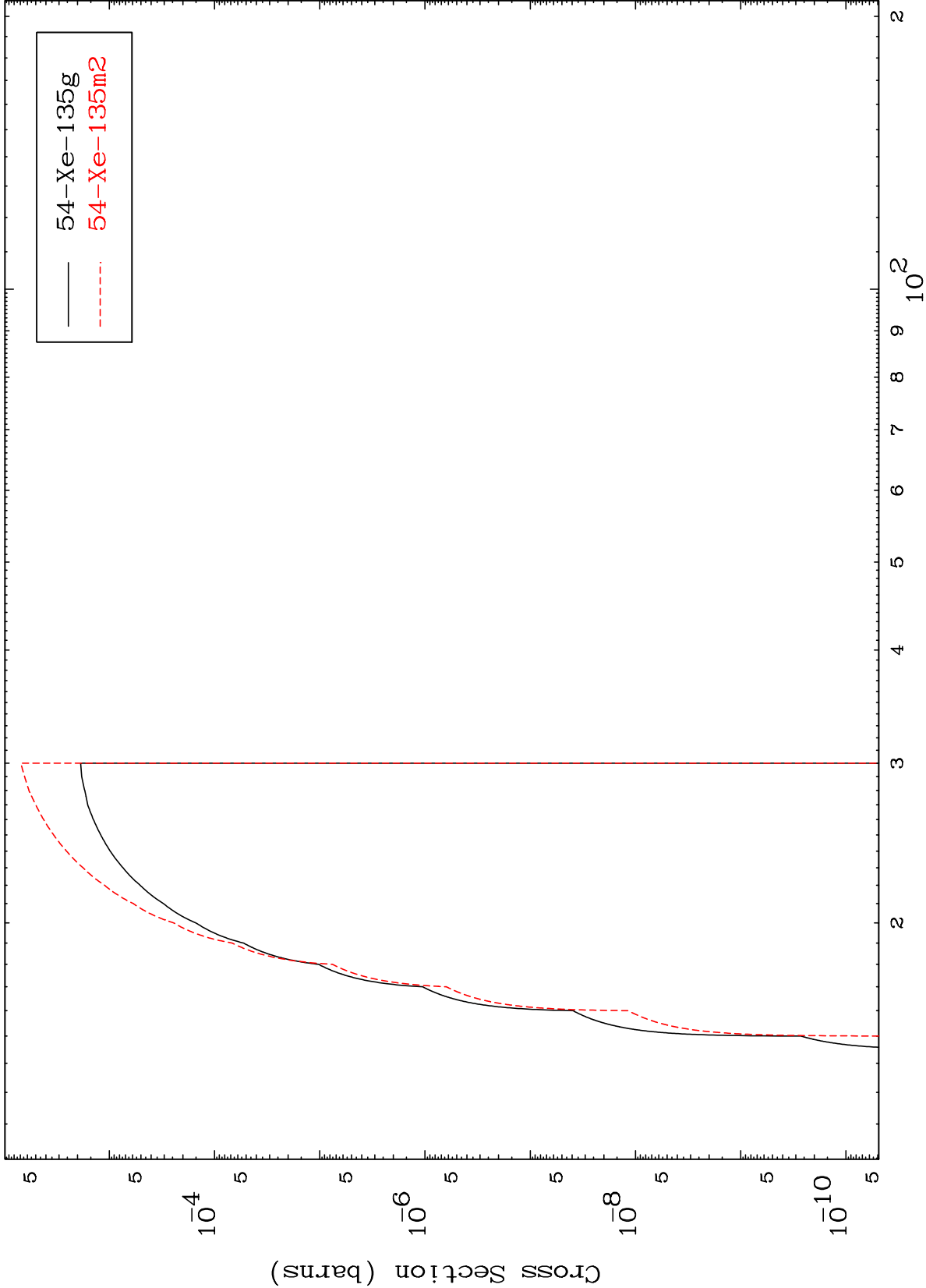


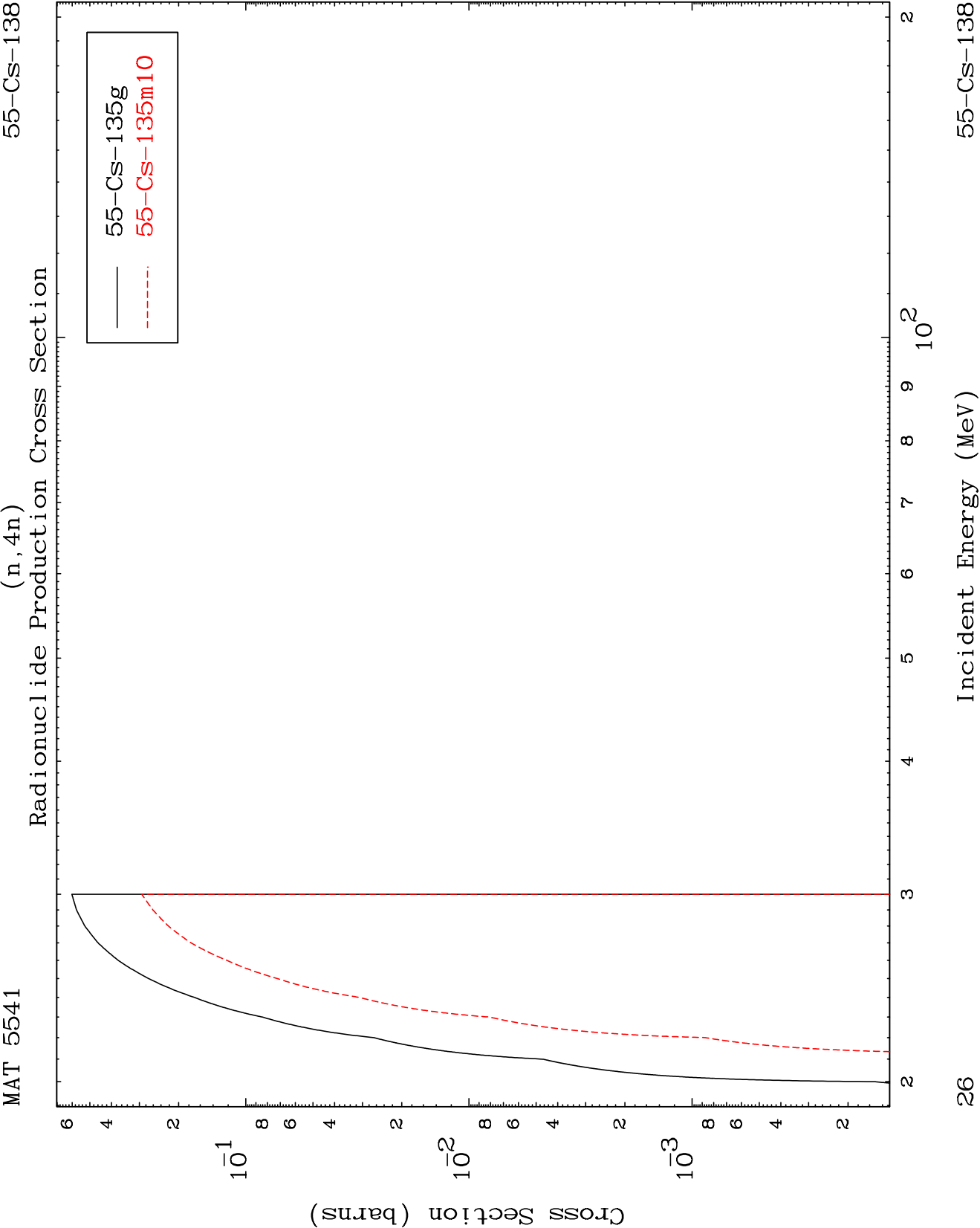
24

Incident Energy (MeV)

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Radionuclide Production Cross Section

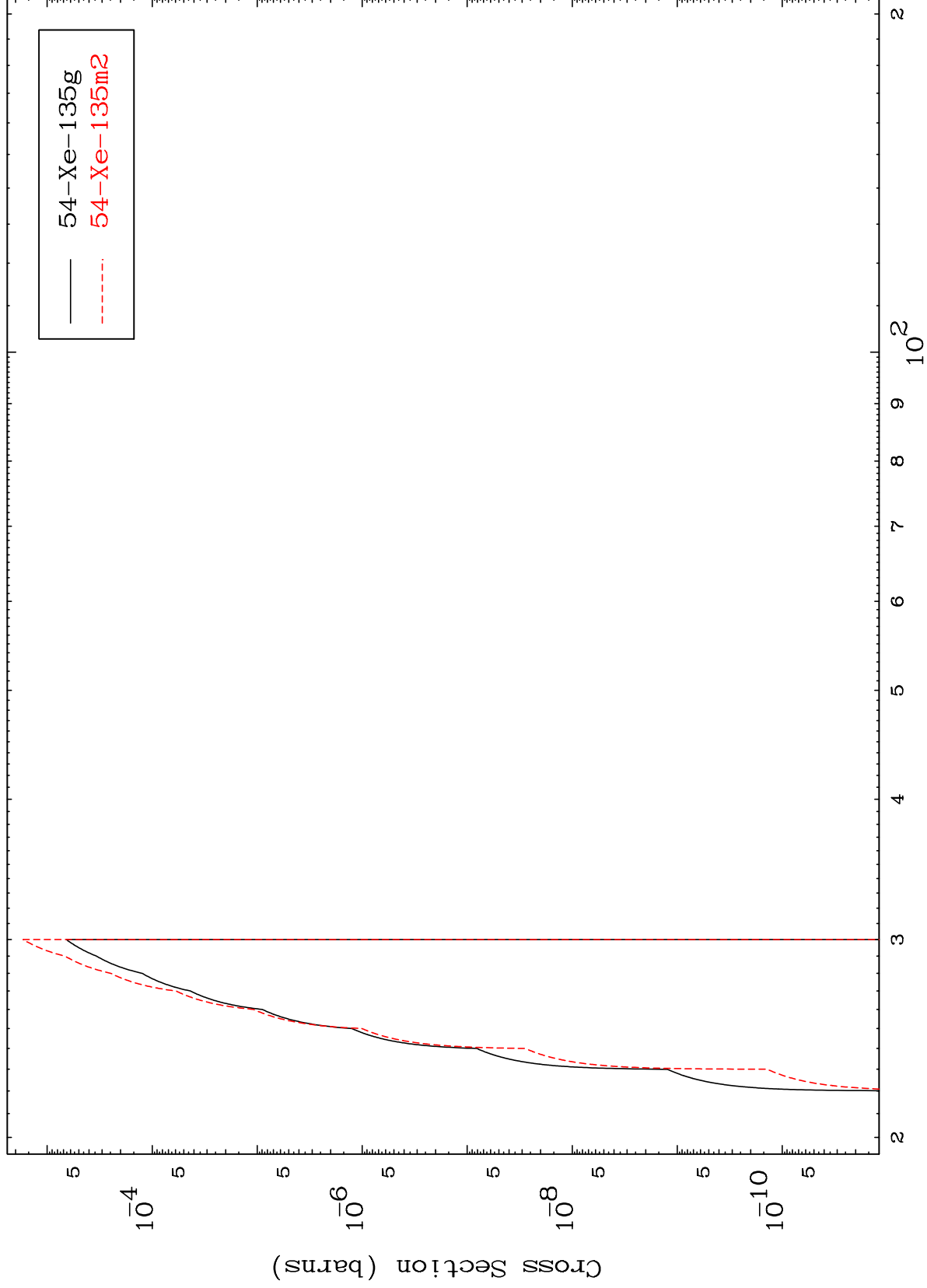




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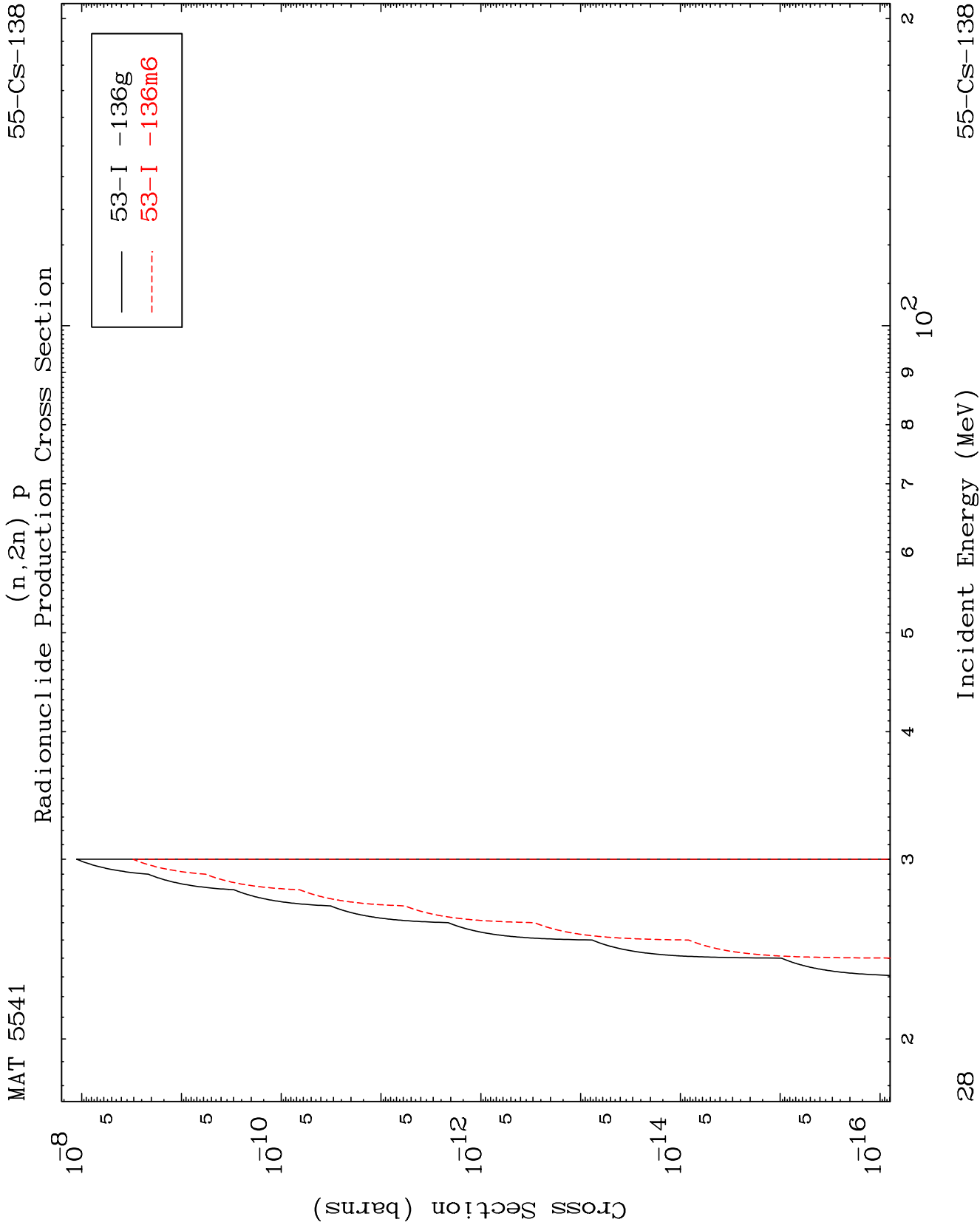
(n,3n) p
Radionuclide Production Cross Section



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Incident Energy (MeV)

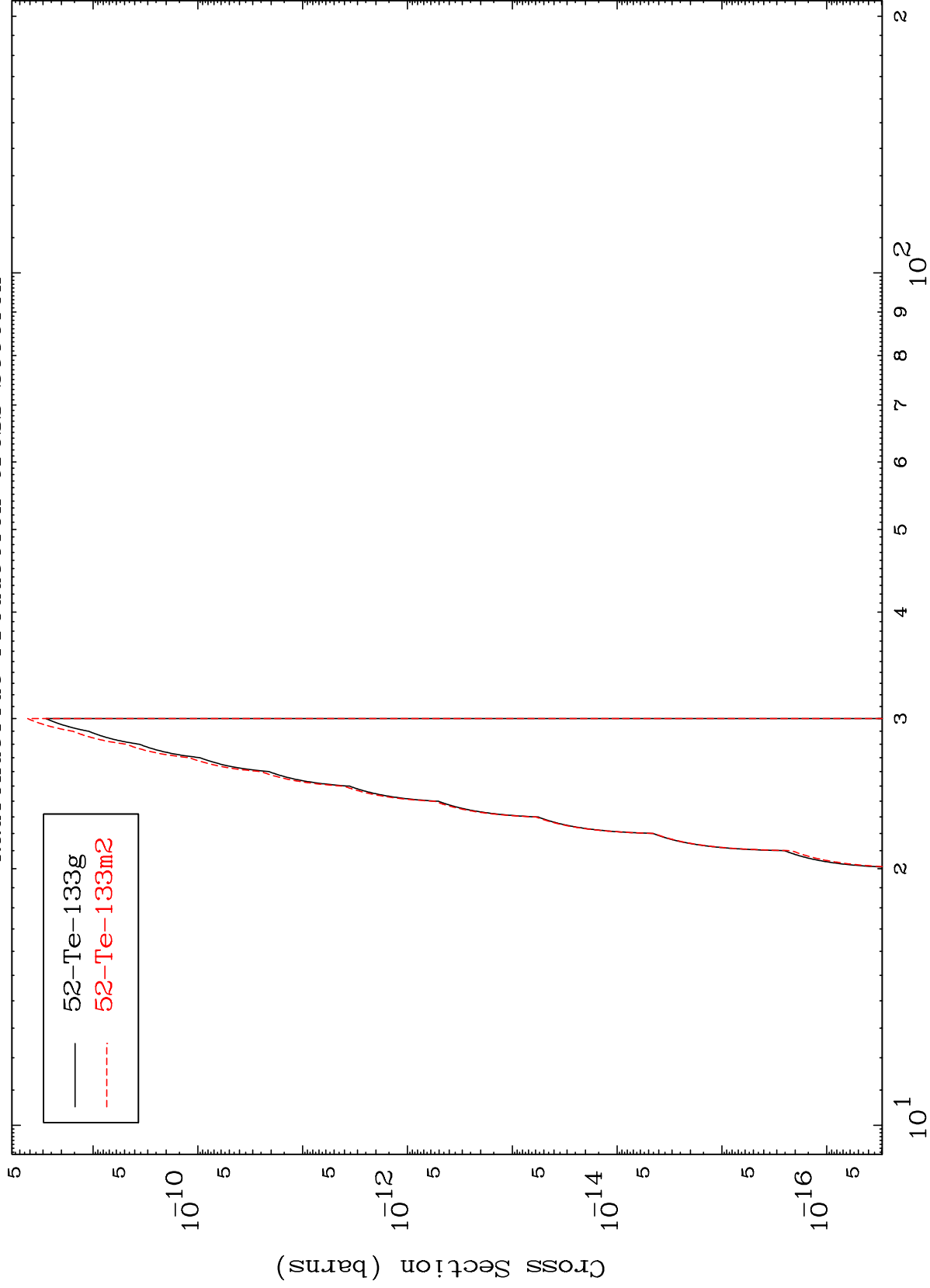
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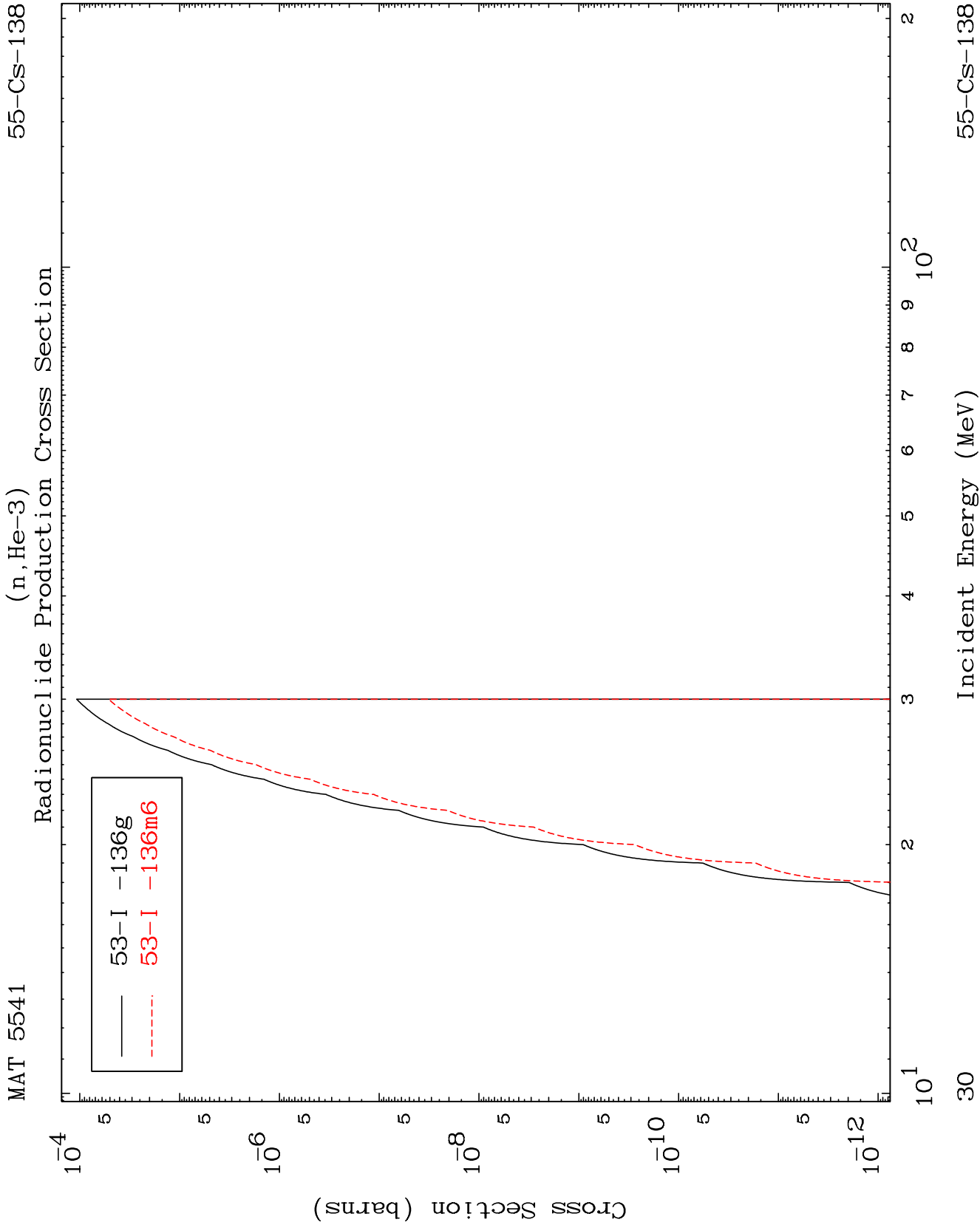
(n,n') p α
Radionuclide Production Cross Section

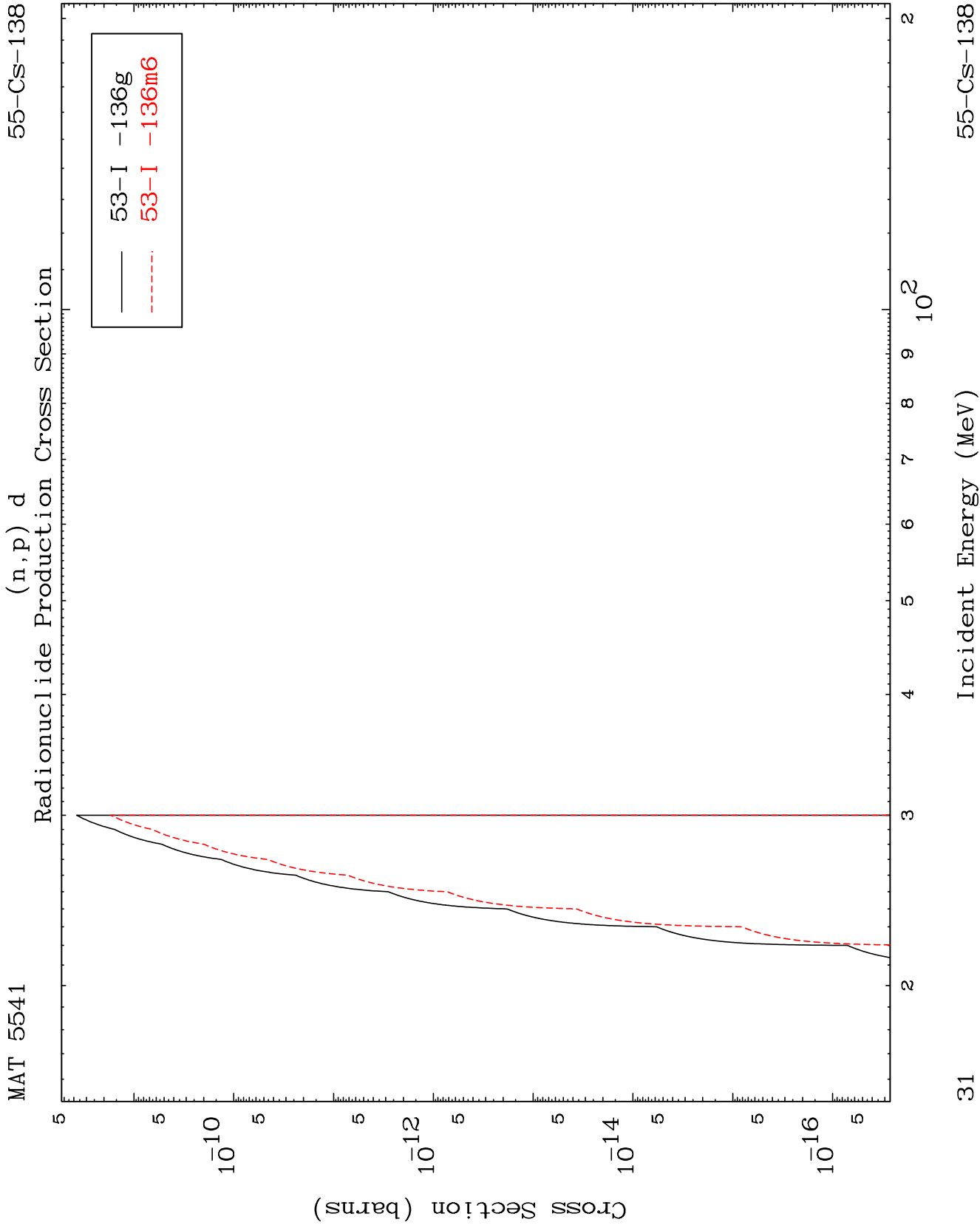


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Incident Energy (MeV)

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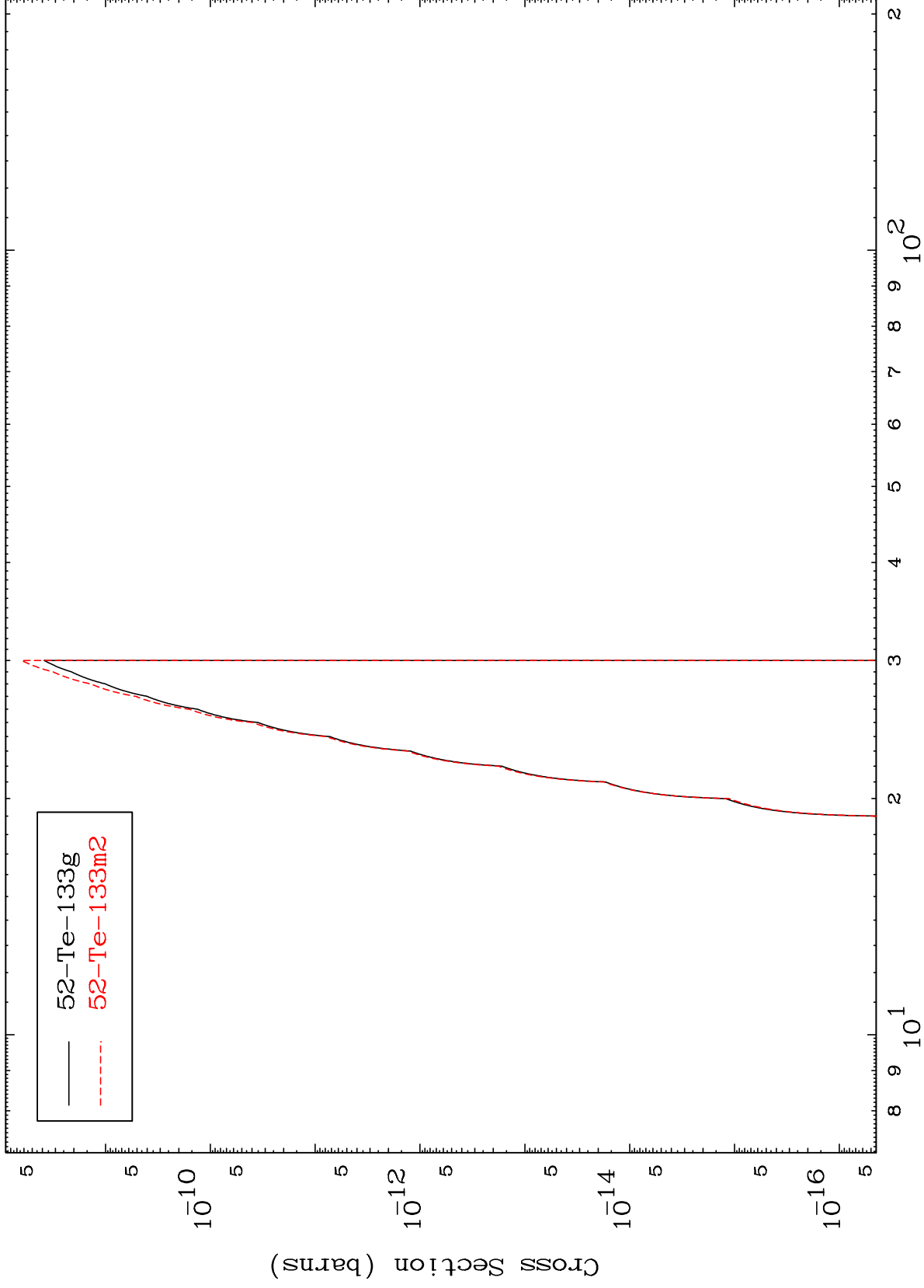




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(n,d) α
Radionuclide Production Cross Section



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Incident Energy (MeV)

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