

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

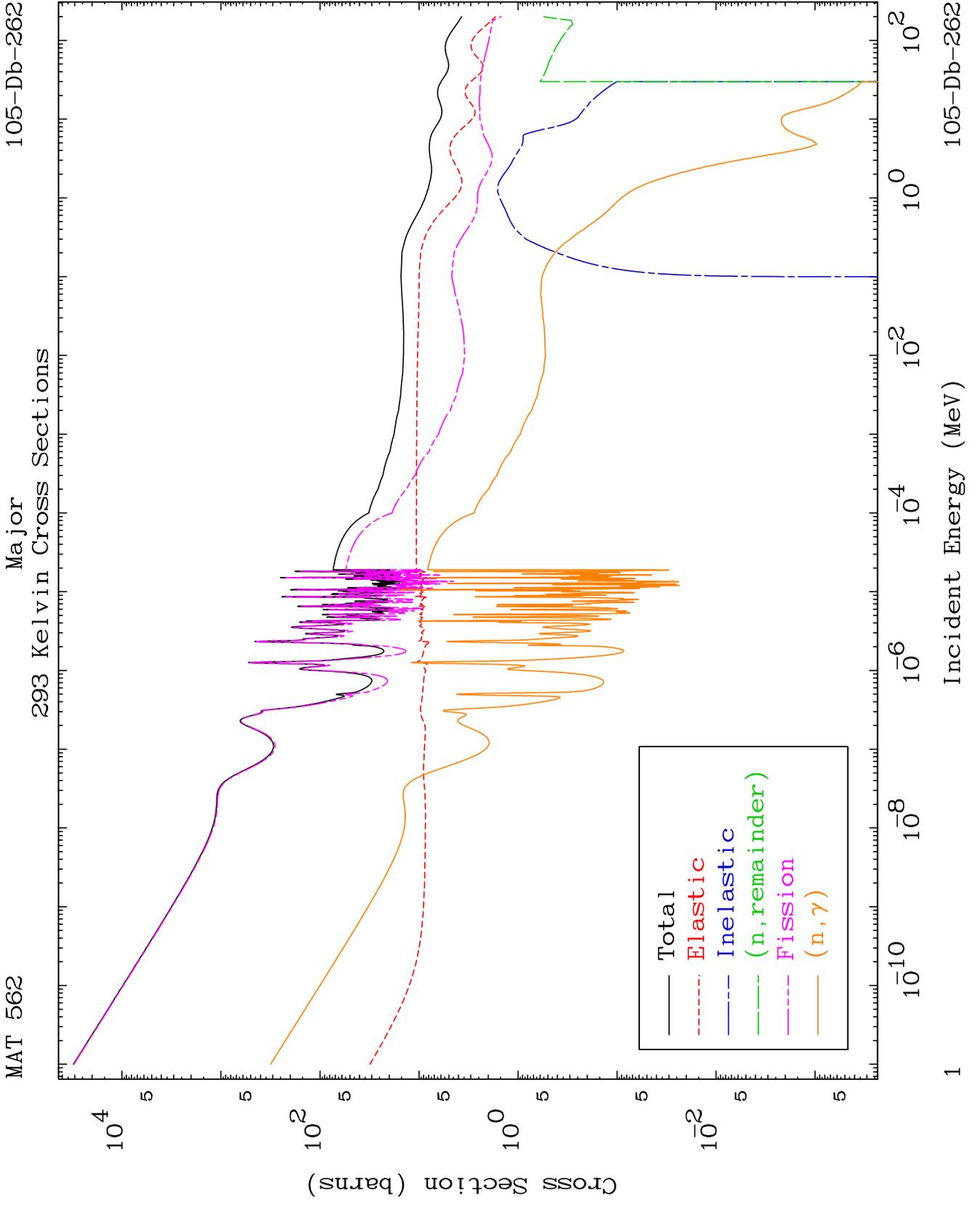
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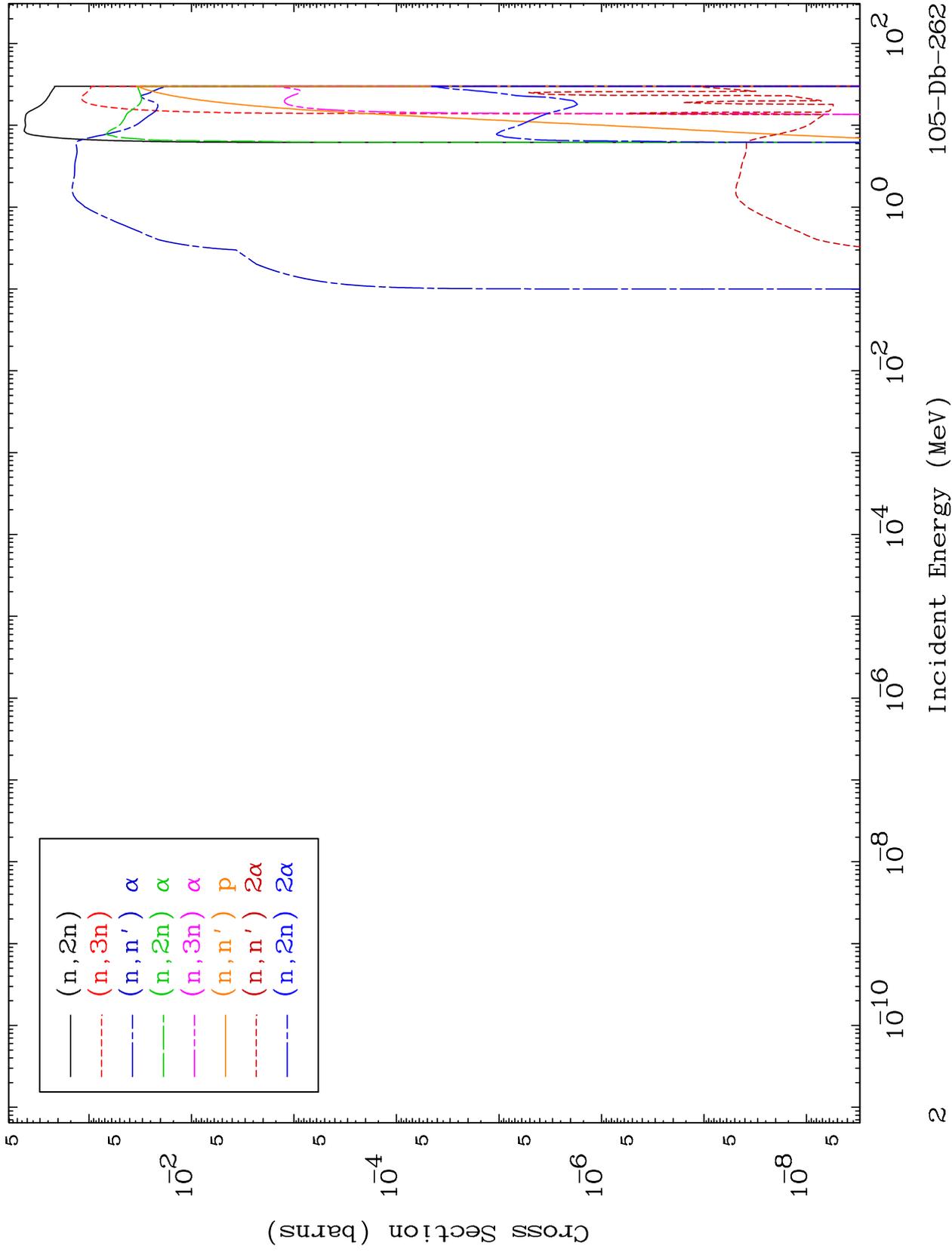
Press Mouse Button to Start



MAT 562

Neutron Production  
293 Kelvin Cross Sections

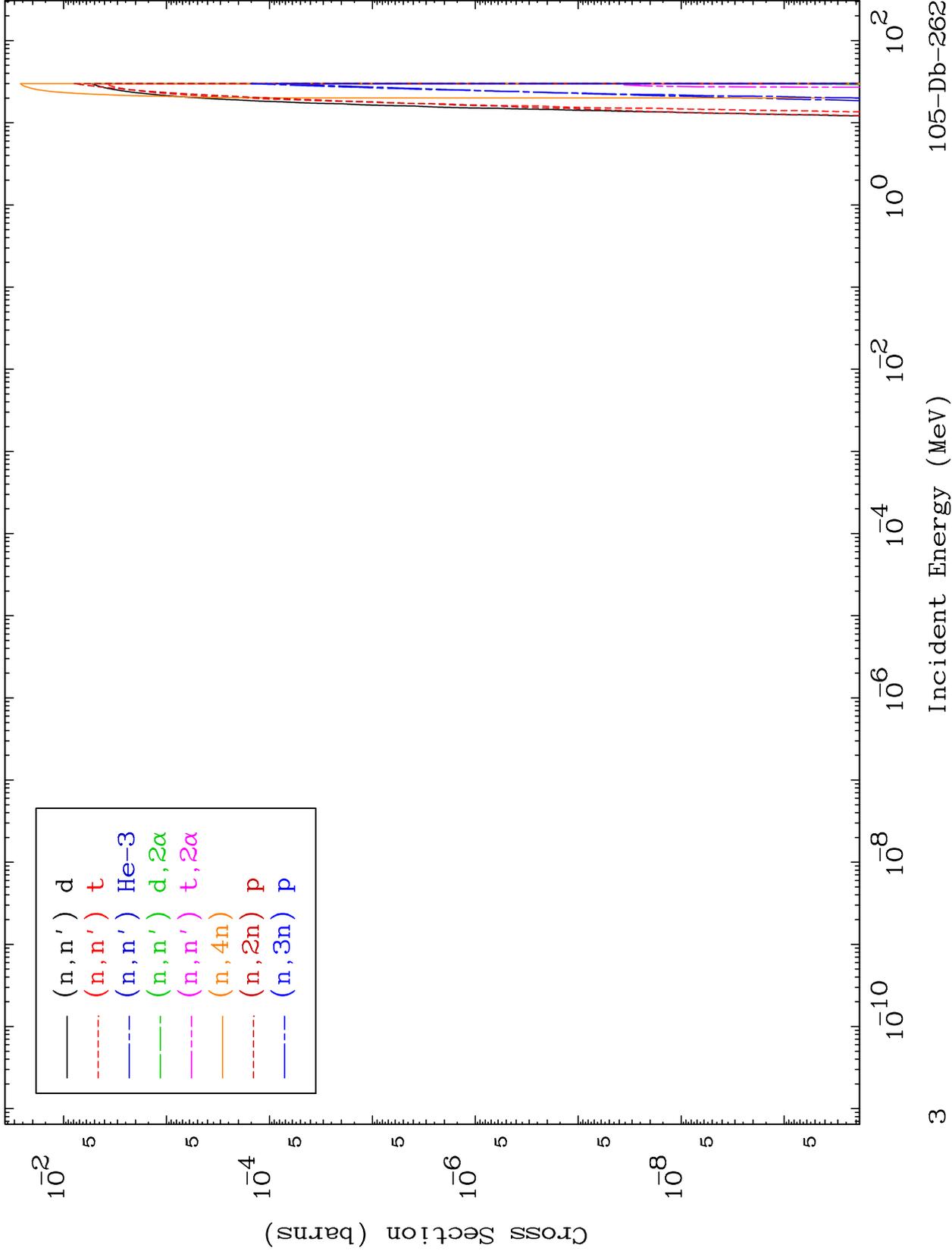
105-Db-262



MAT 562

Neutron Production  
293 Kelvin Cross Sections

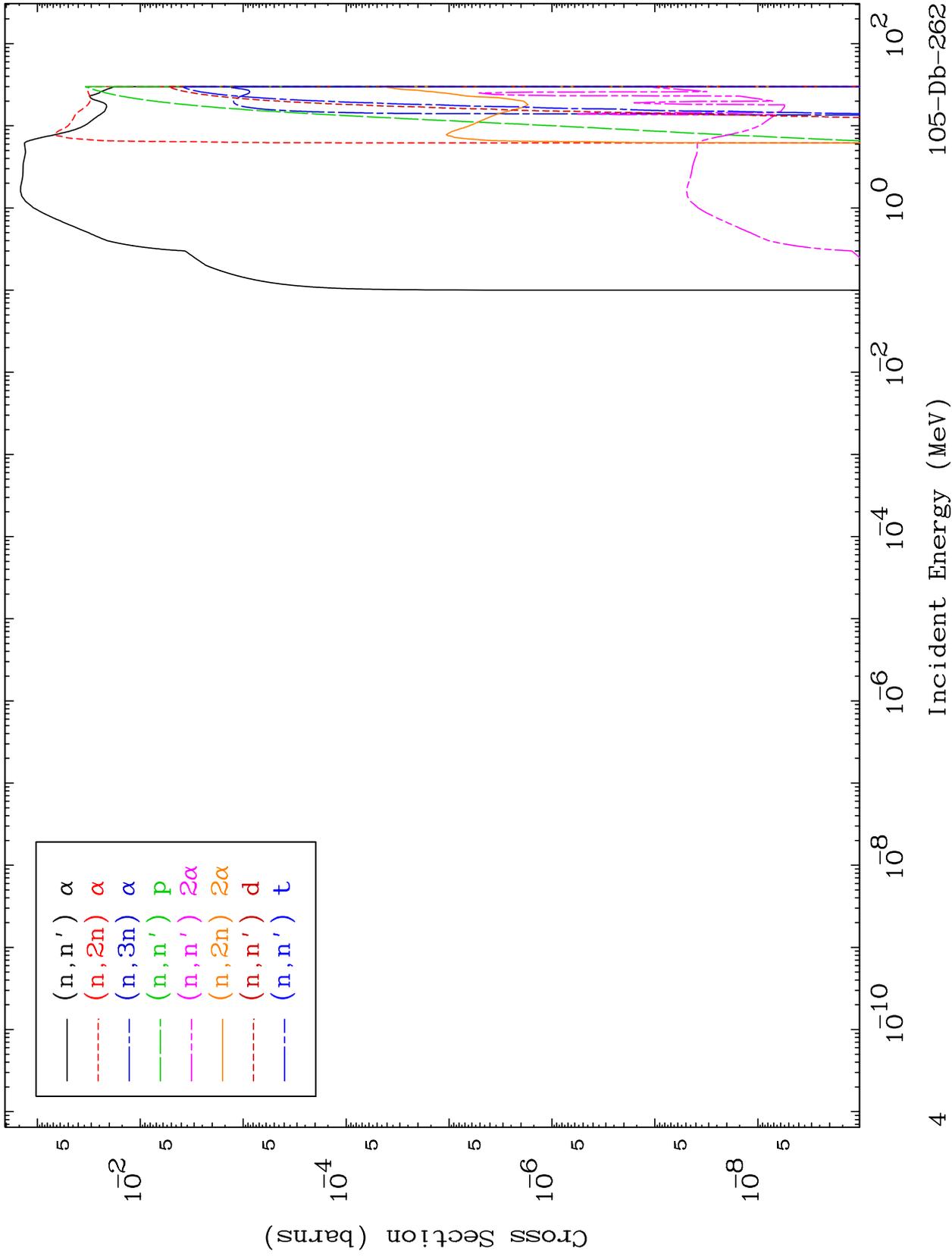
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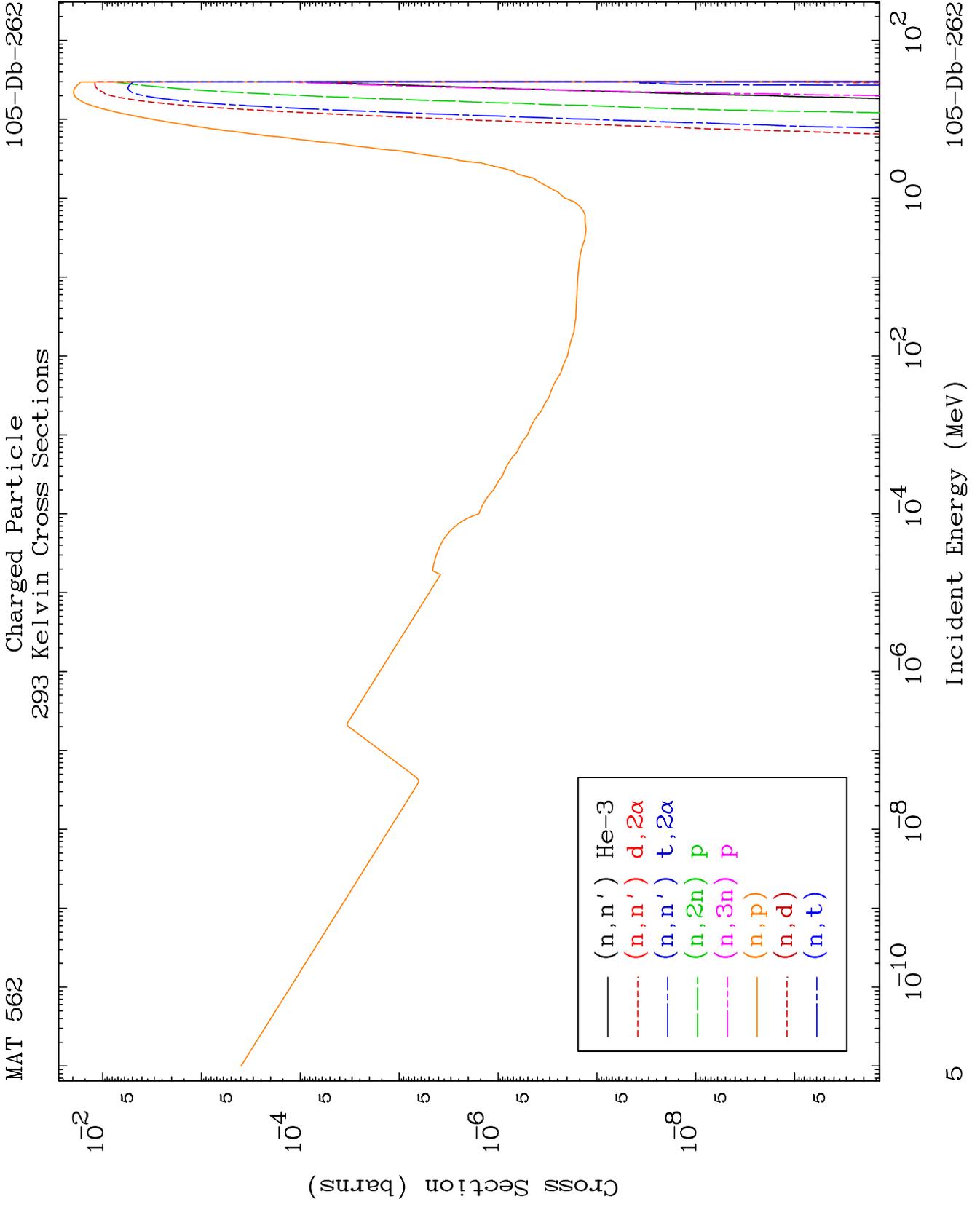


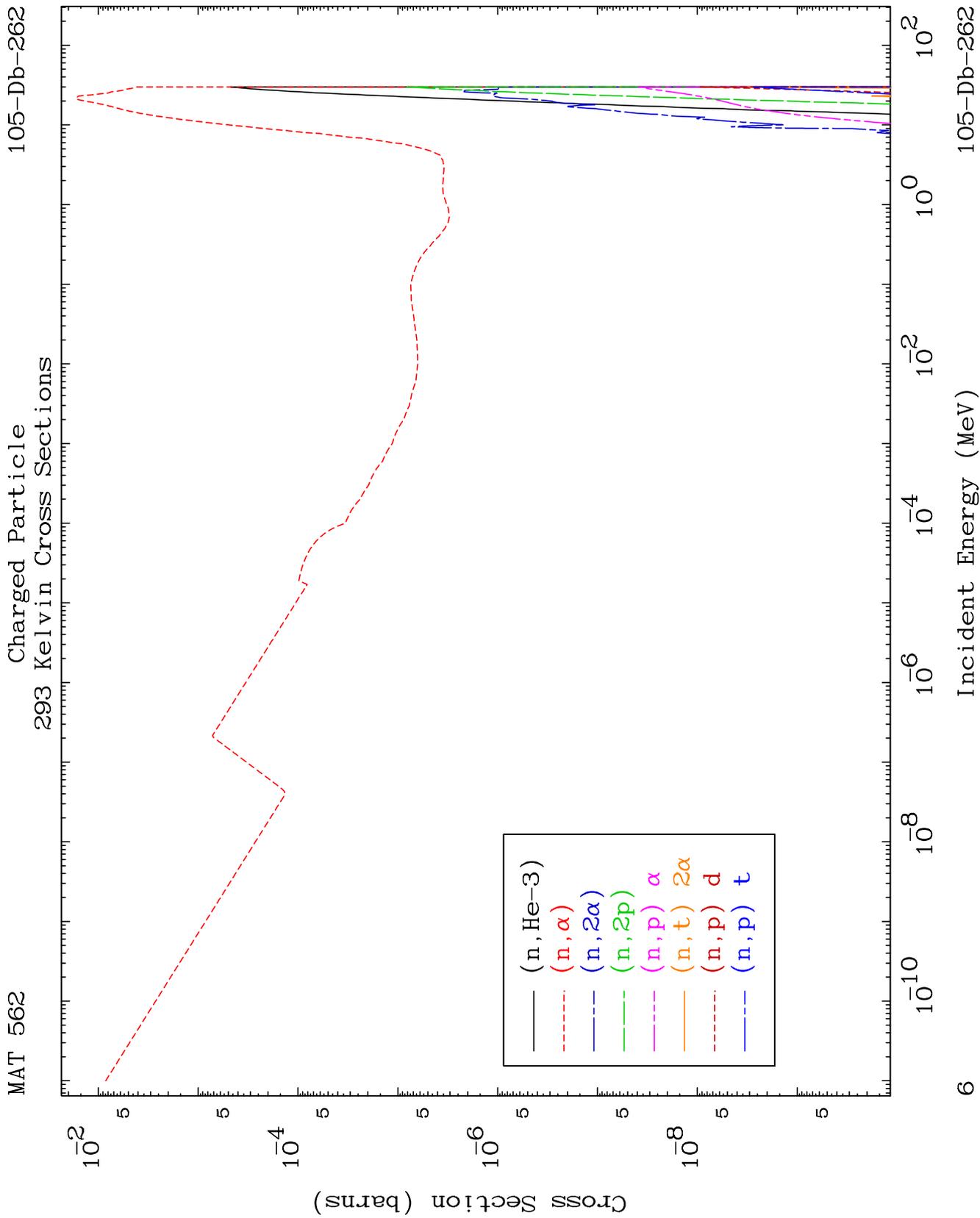
MAT 562

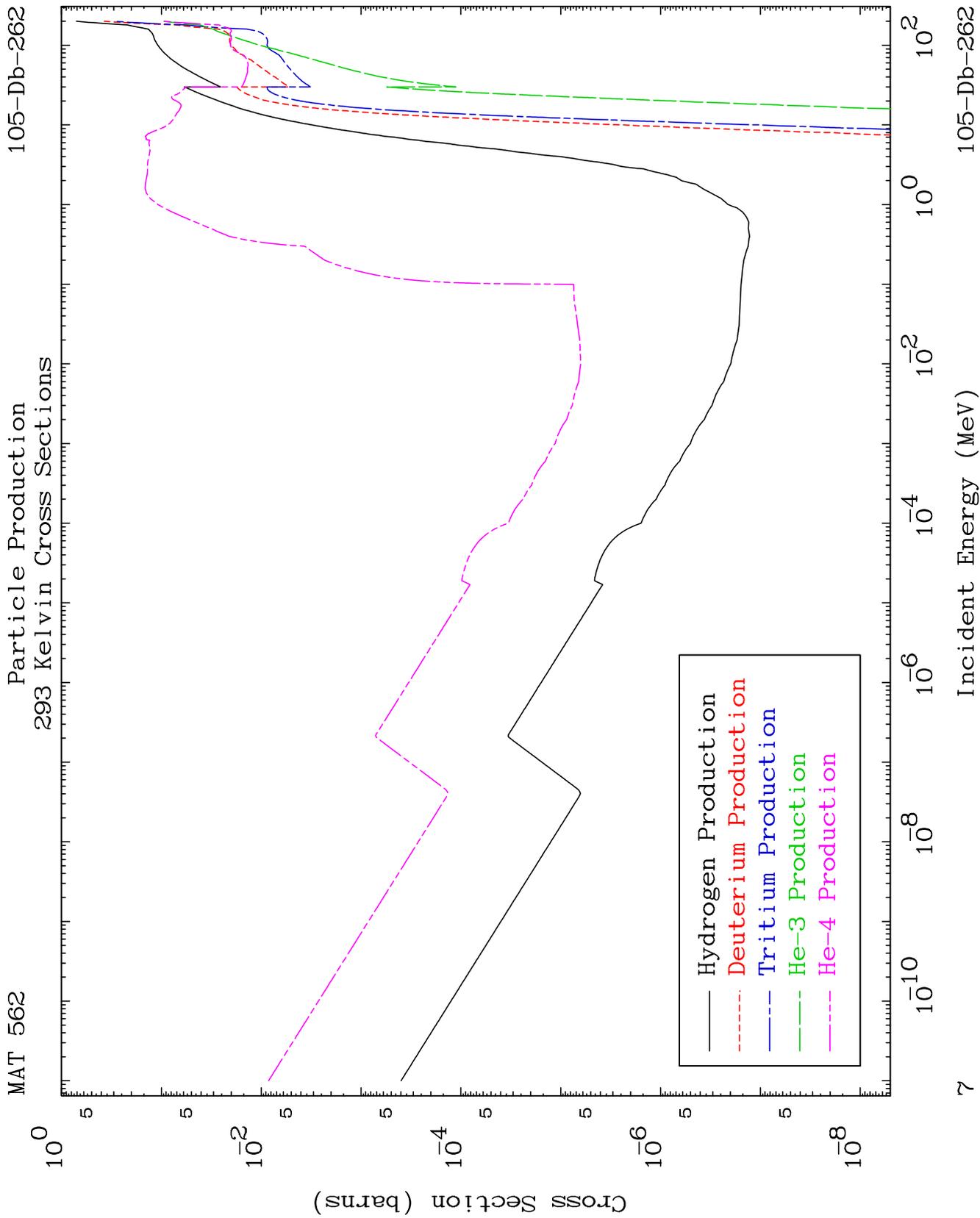
Charged Particle  
293 Kelvin Cross Sections

105-Db-262





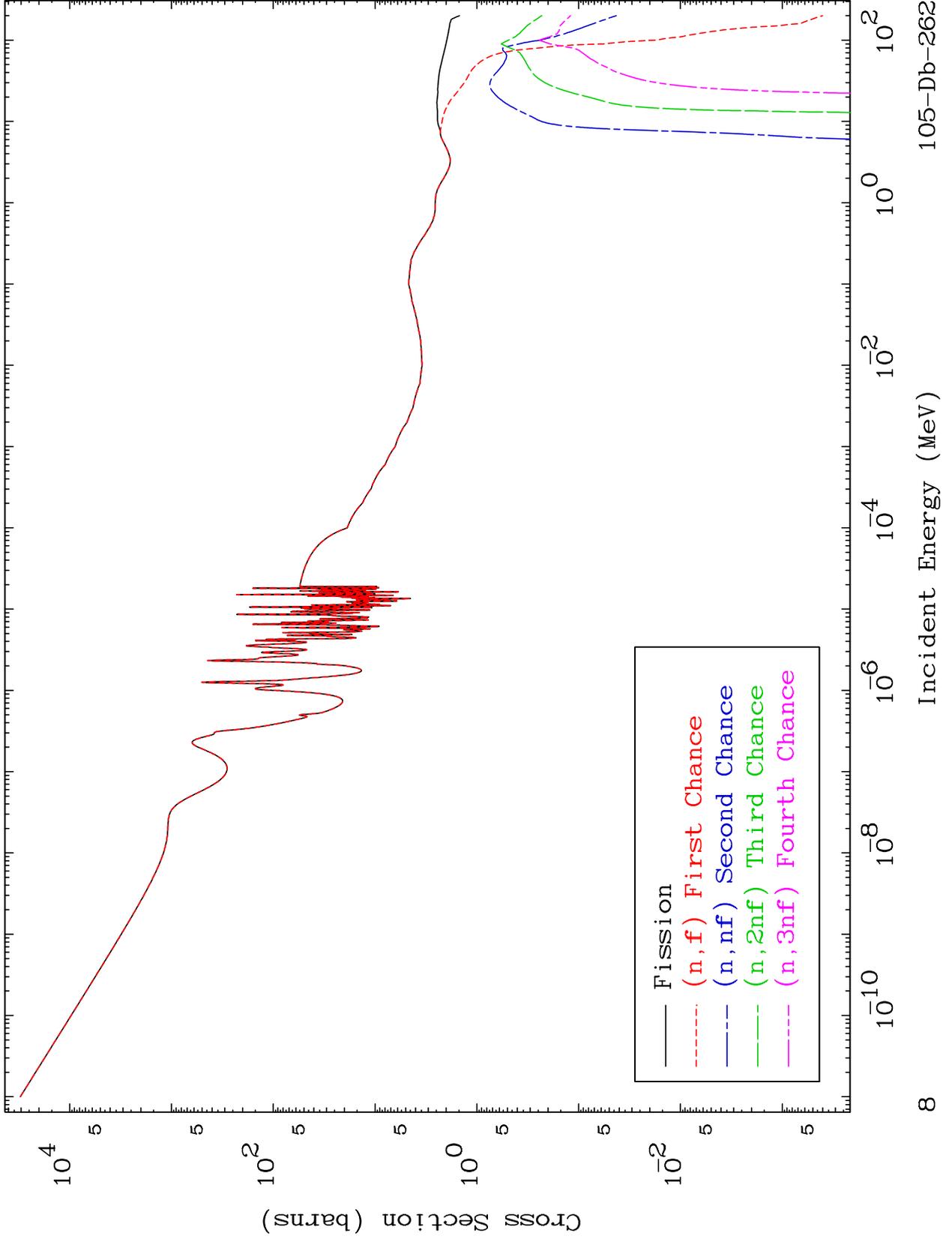




MAT 562

Fission  
293 Kelvin Cross Sections

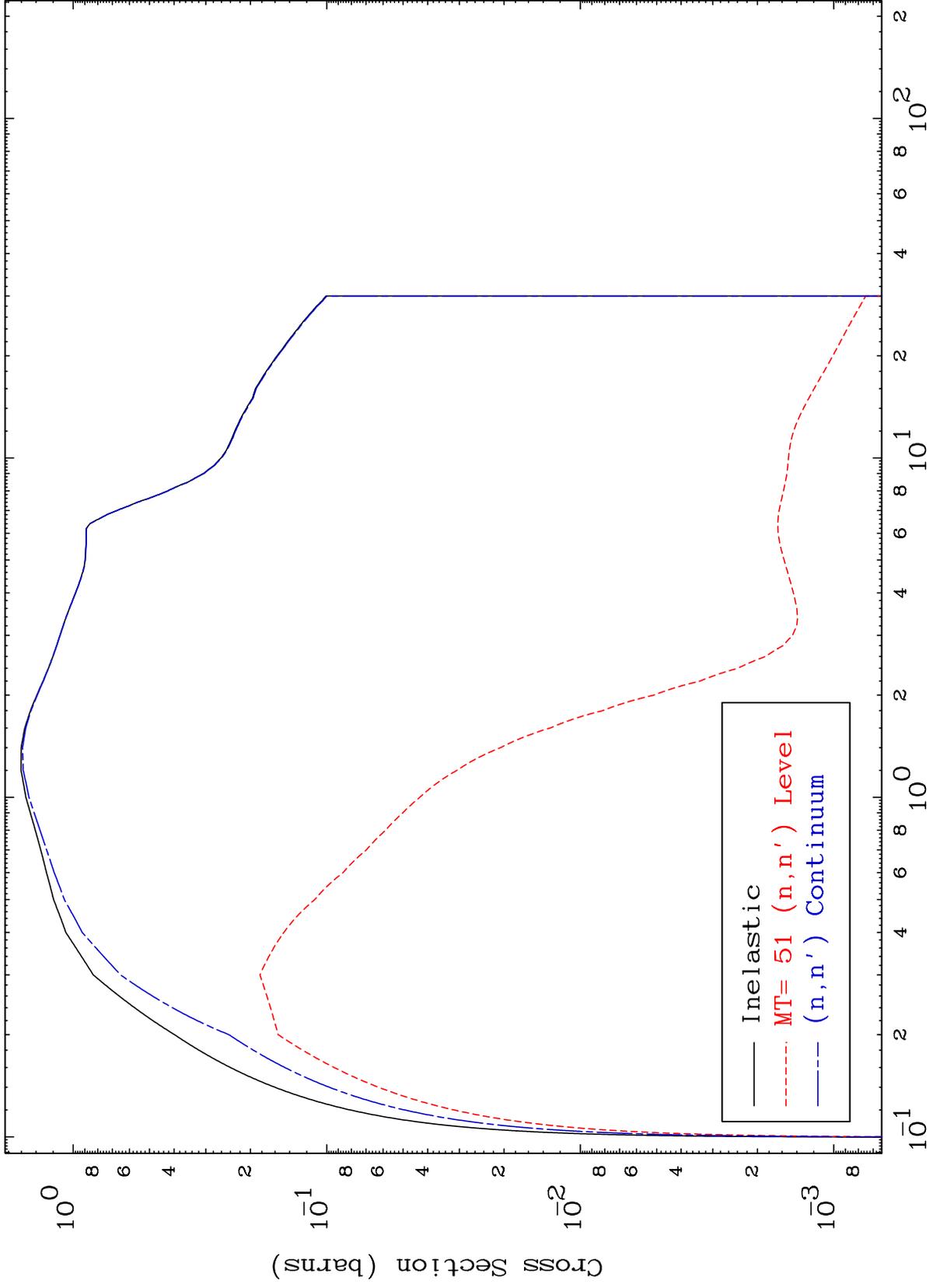
105-Db-262



MAT 562

(n,n') Level  
293 Kelvin Cross Sections

105-Db-262



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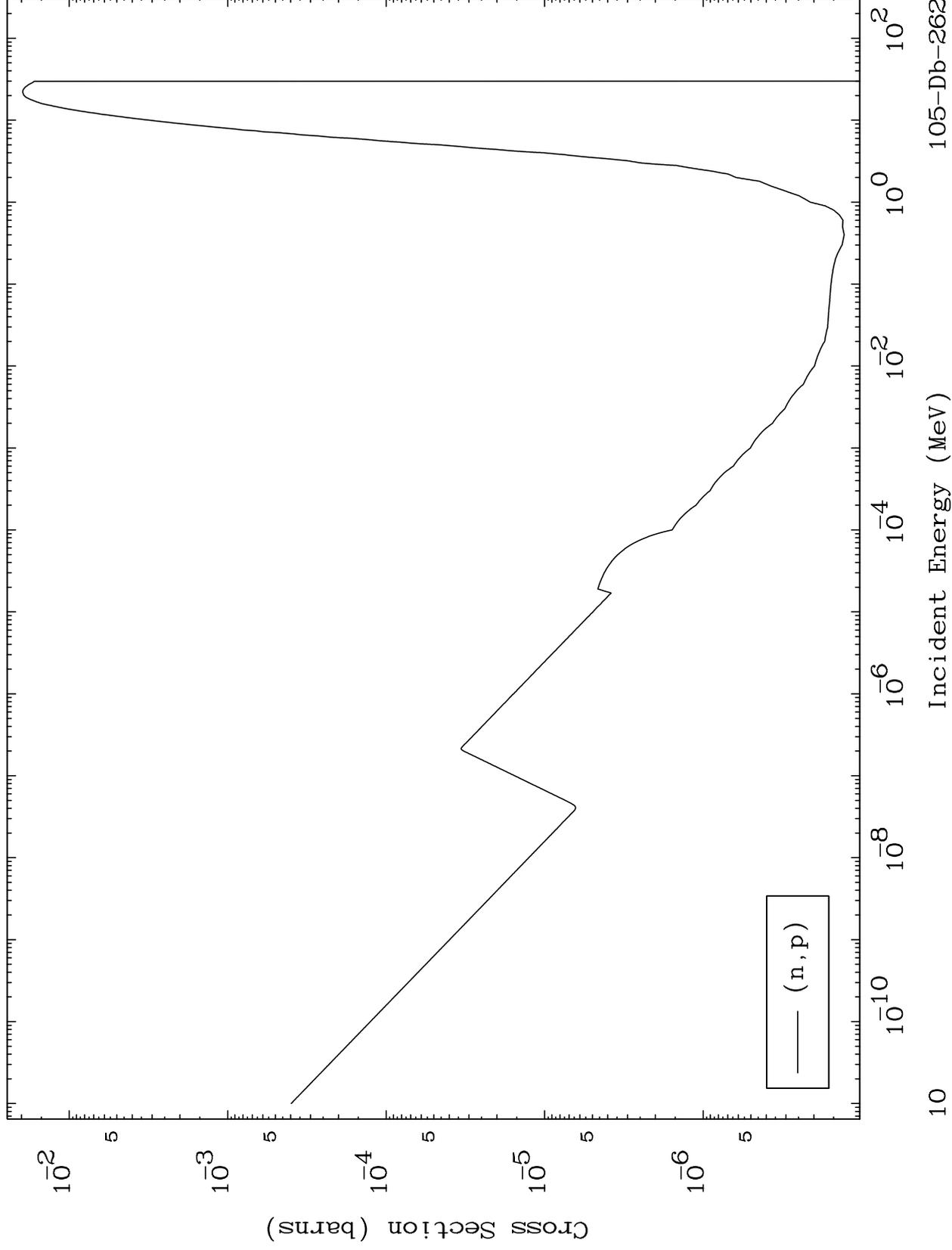
Incident Energy (MeV)

105-Db-262

MAT 562

(n,p) Levels  
293 Kelvin Cross Sections

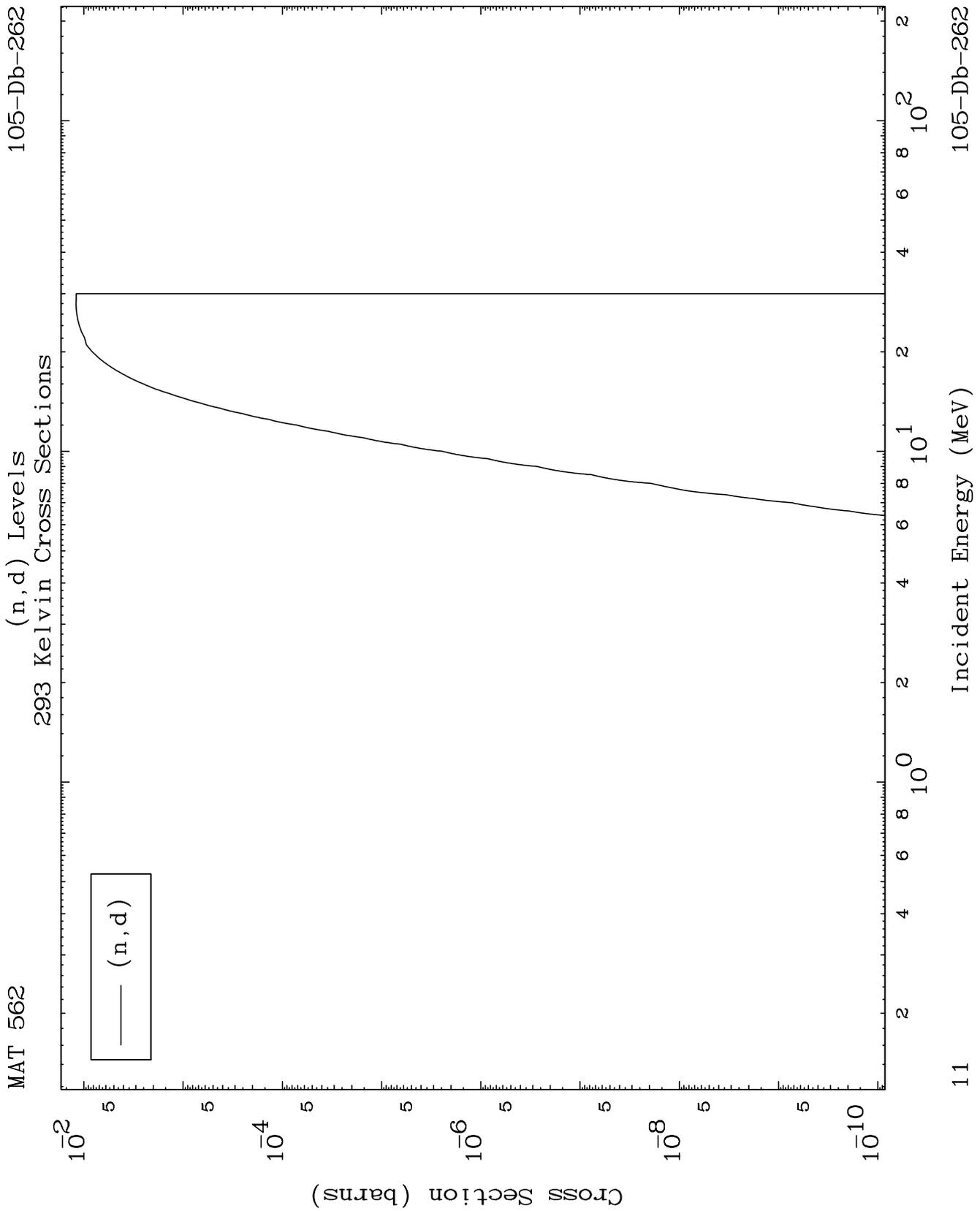
105-Db-262



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Incident Energy (MeV)

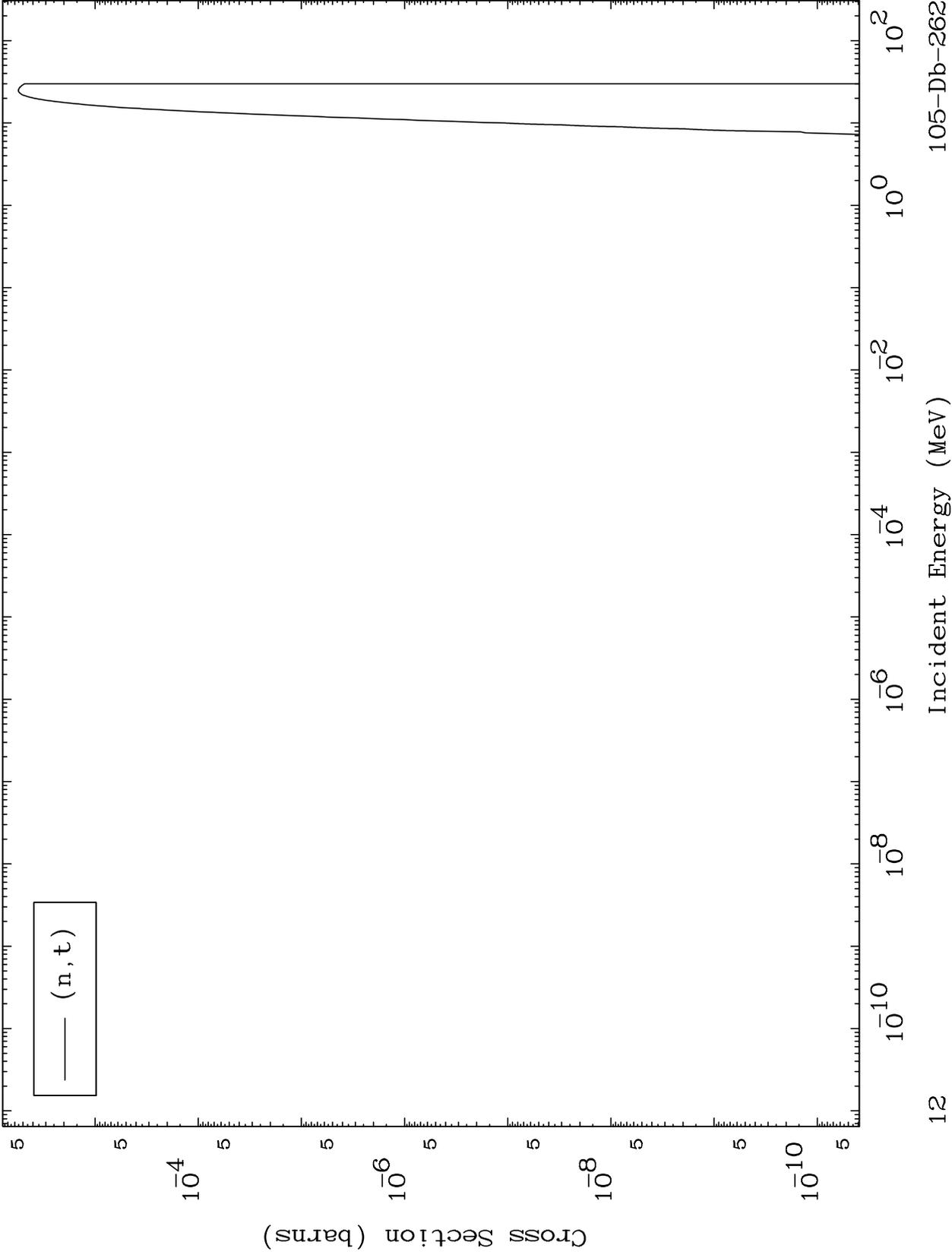
105-Db-262



MAT 562

(n,t) Levels  
293 Kelvin Cross Sections

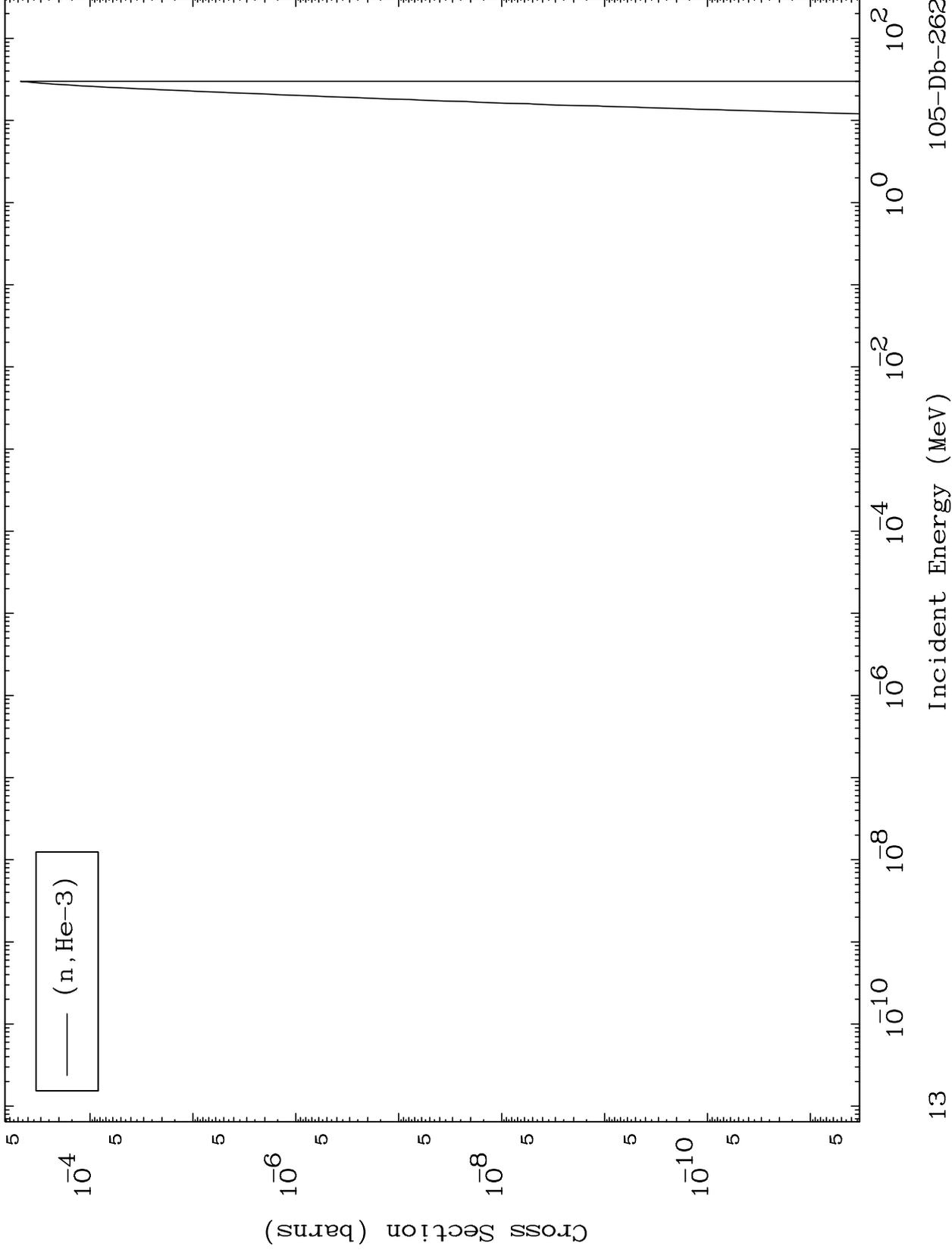
105-Db-262



MAT 562

(n,He3) Levels  
293 Kelvin Cross Sections

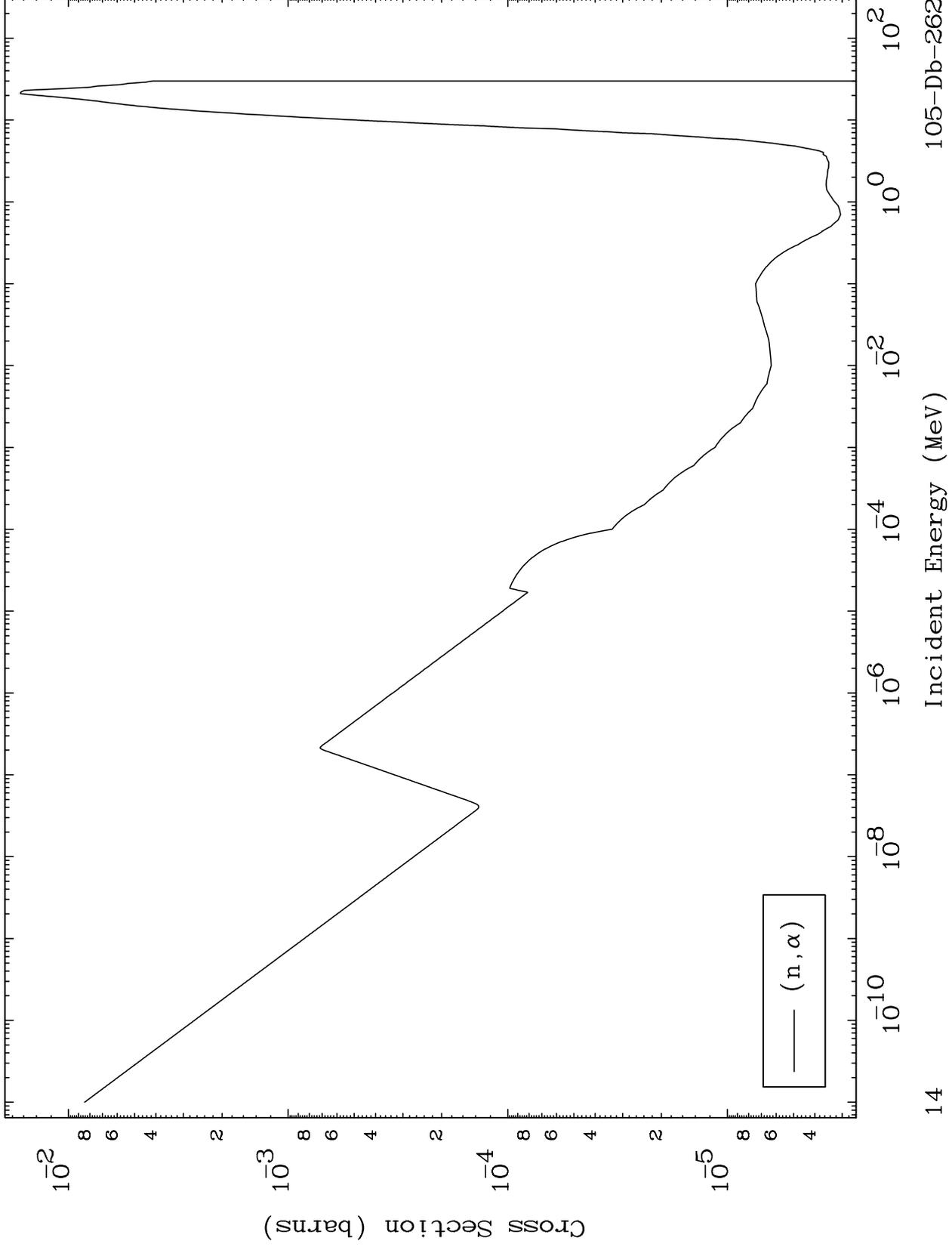
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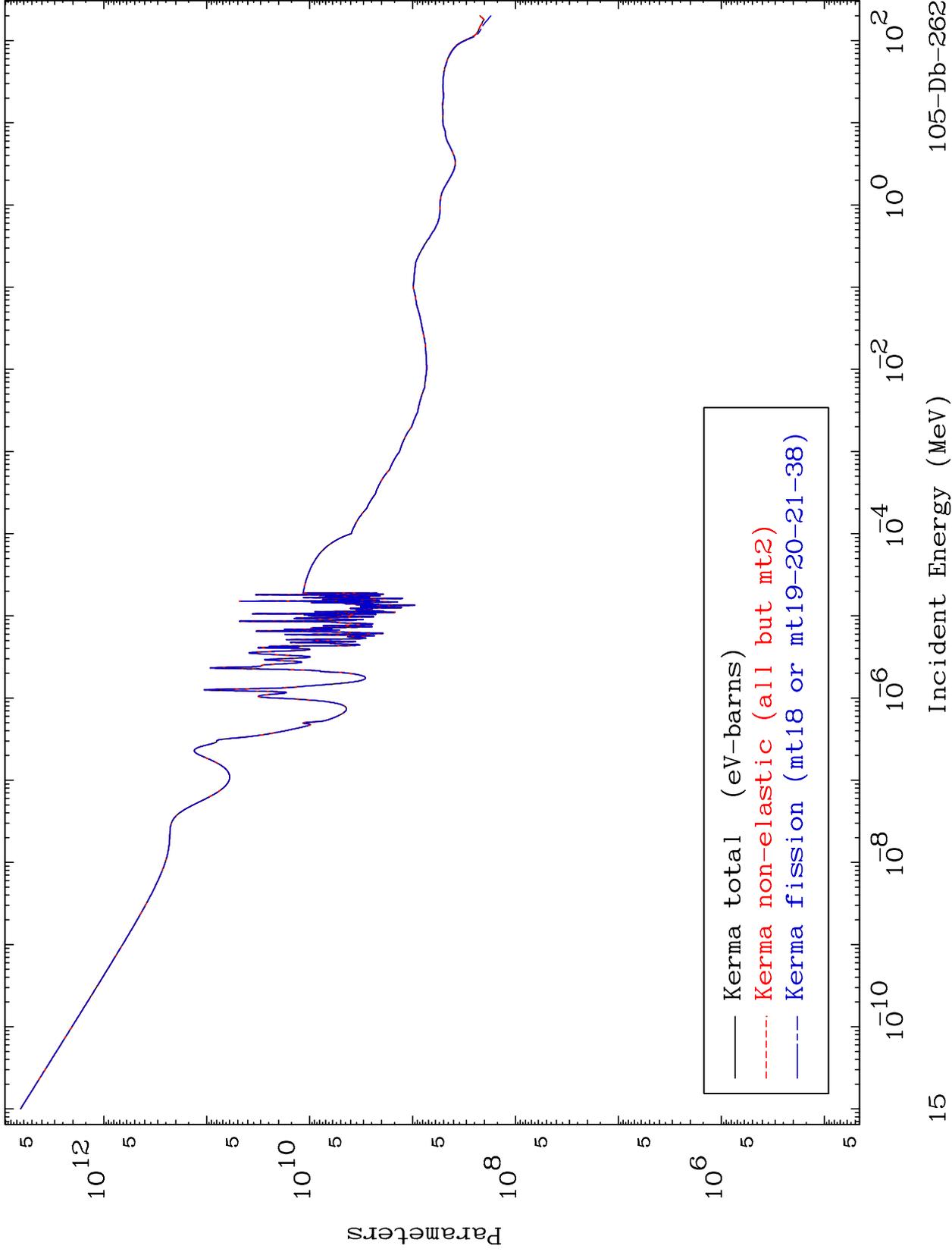


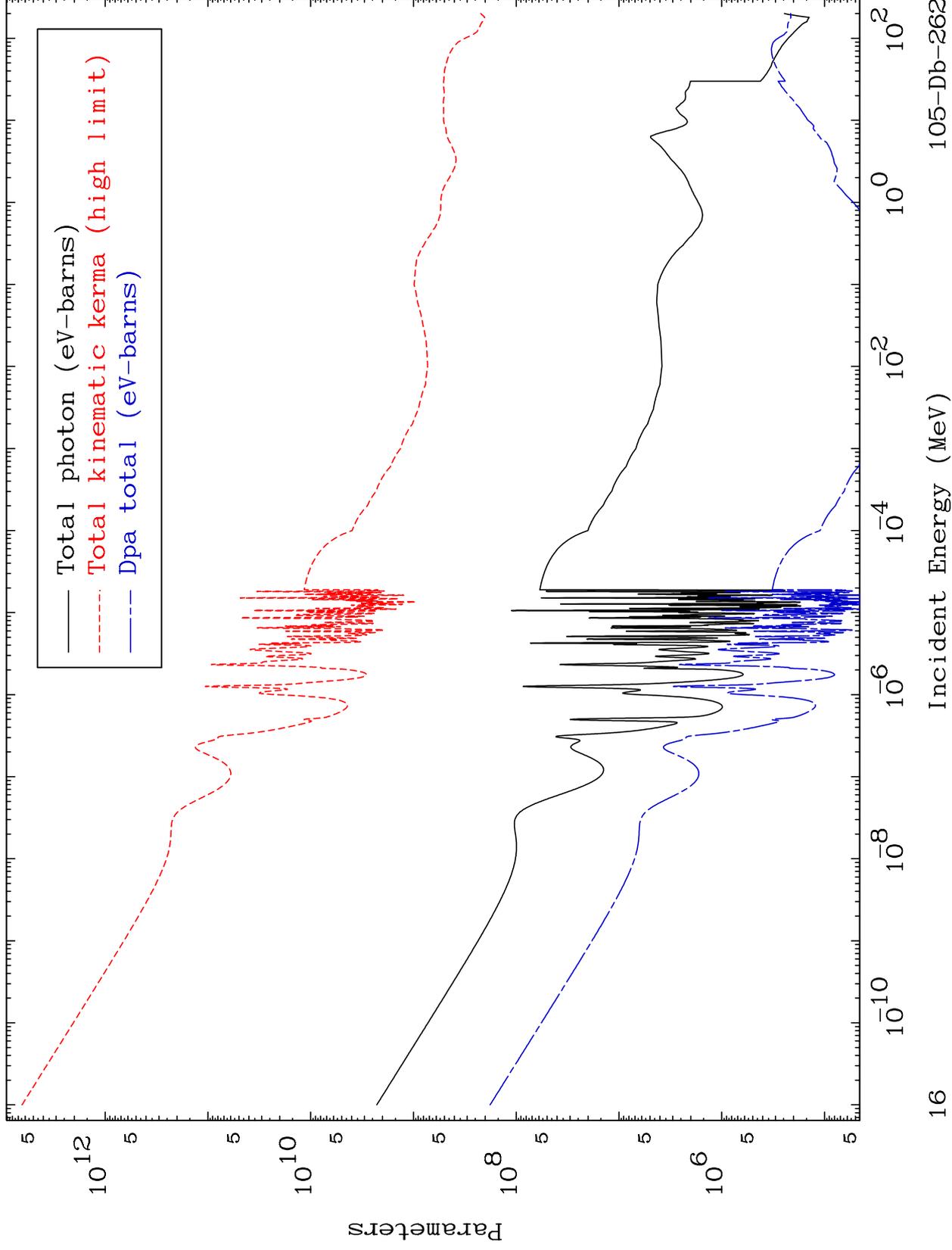
MAT 562

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

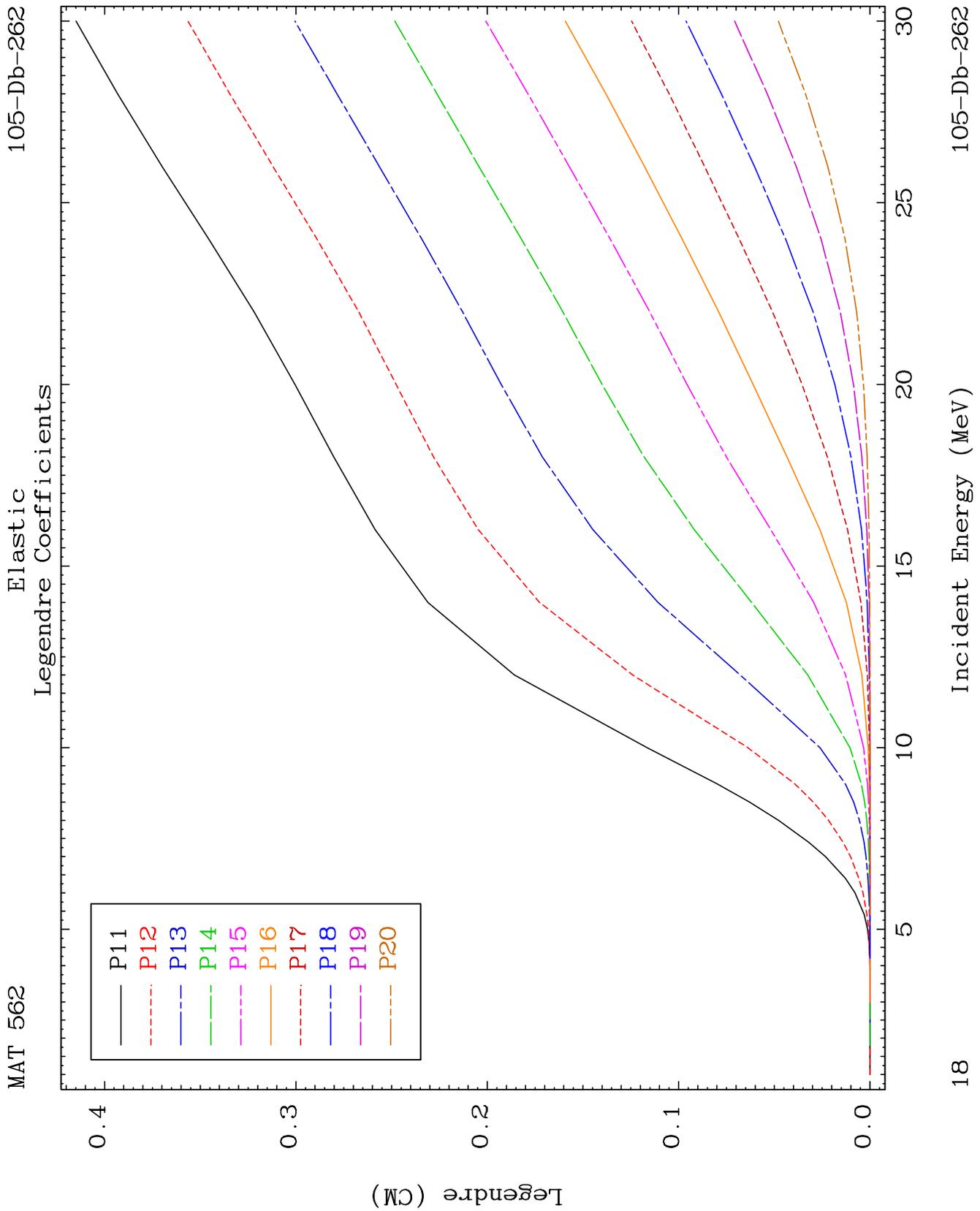
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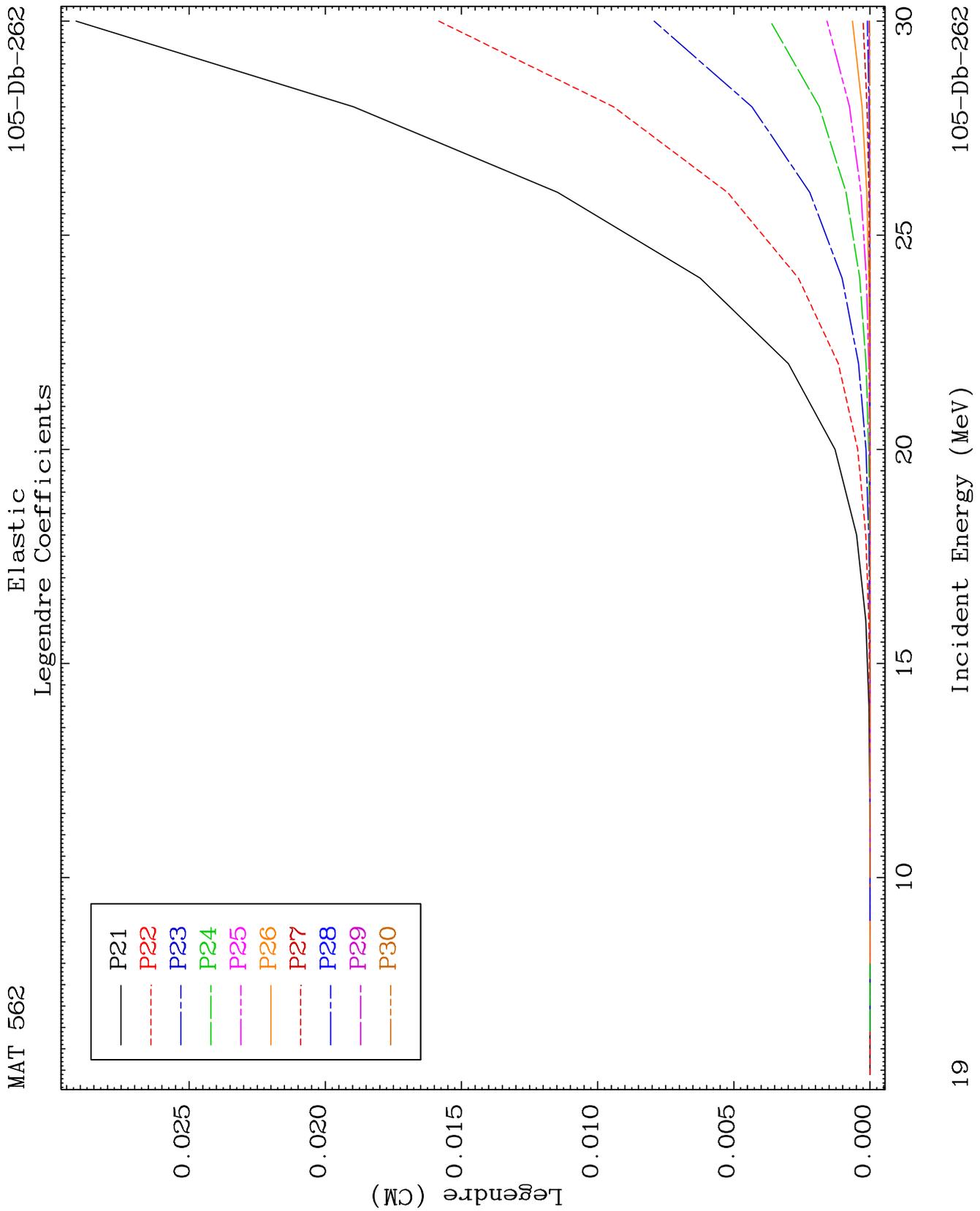










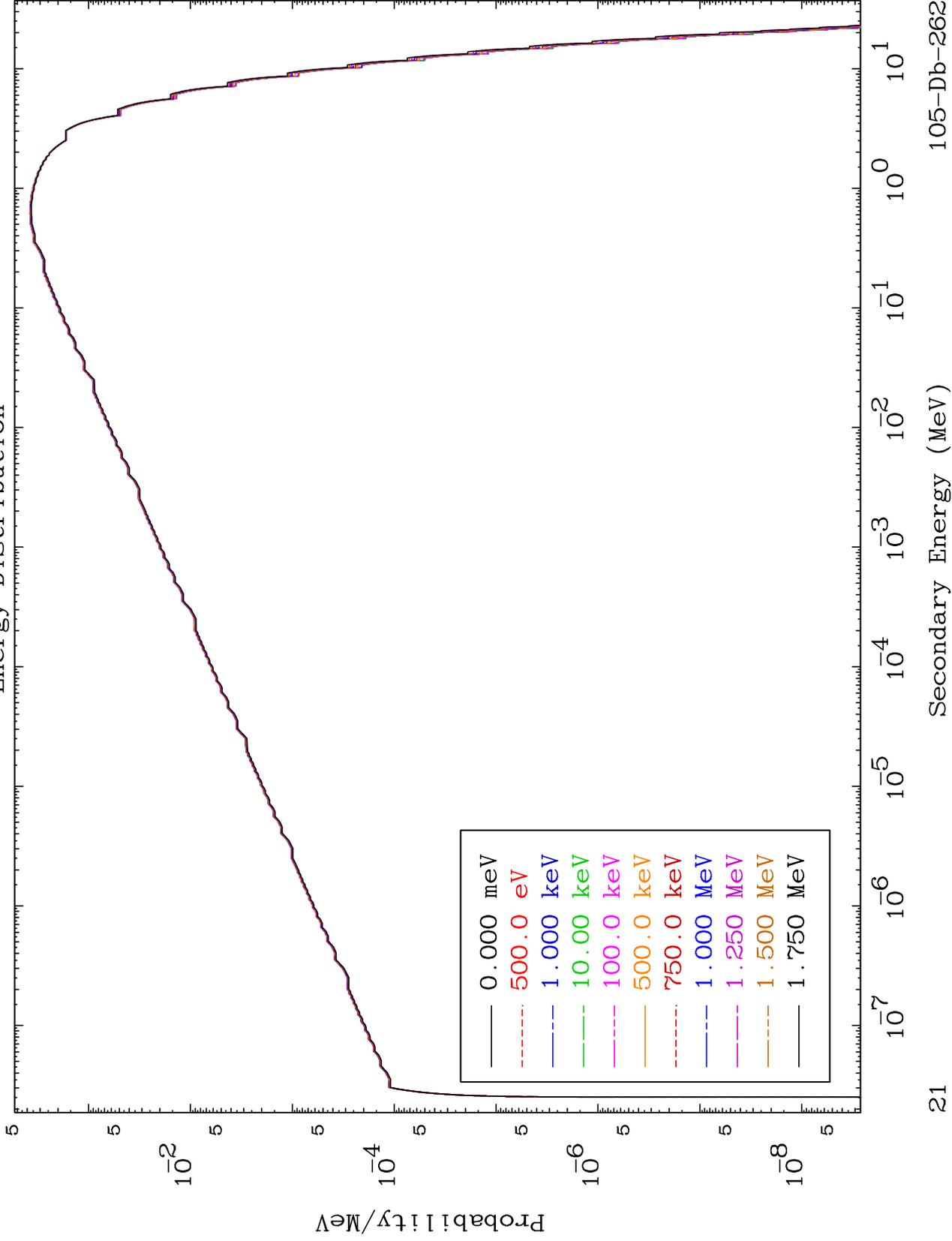




MAT 562

# Fission Energy Distribution

105-Db-262



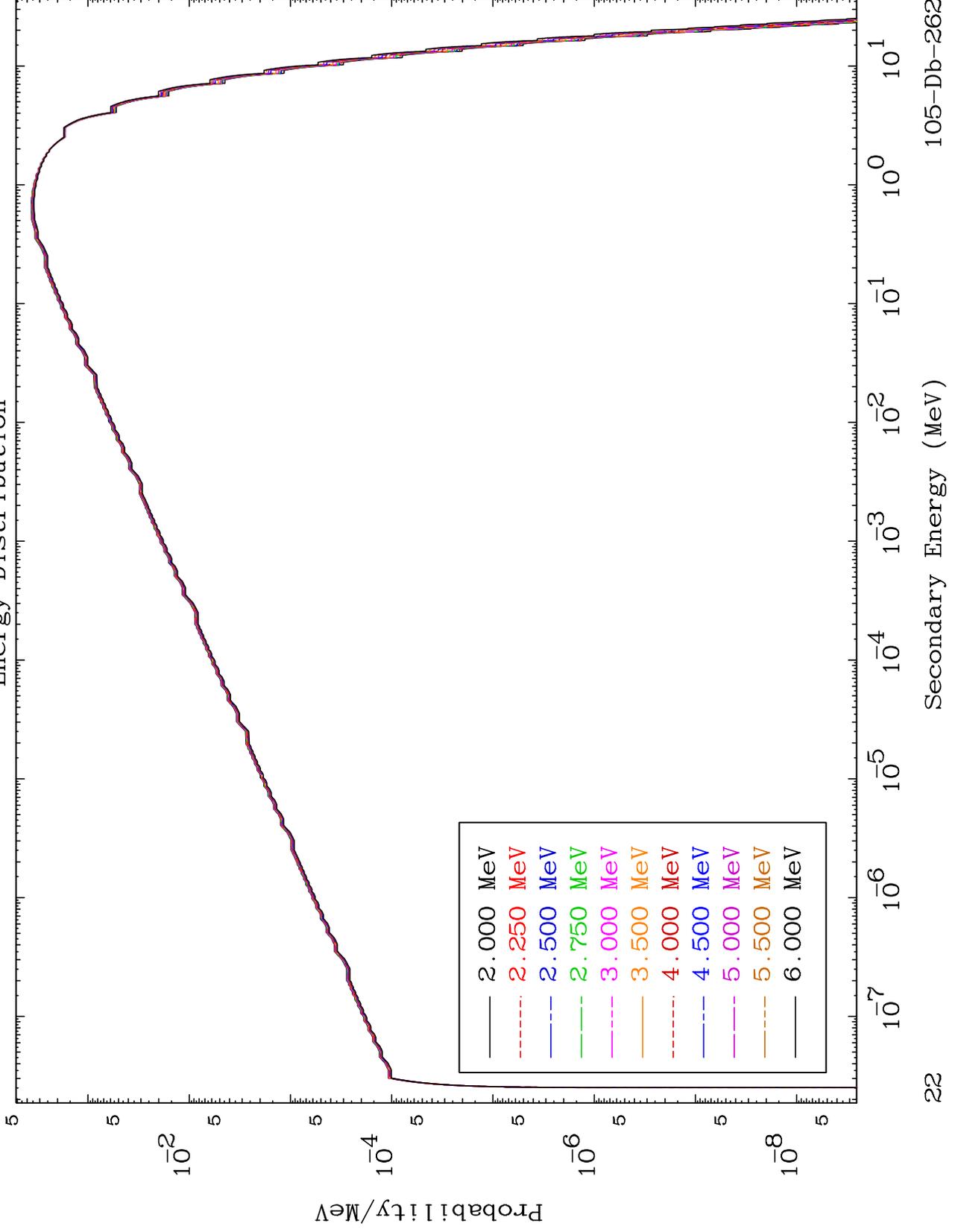
105-Db-262

Secondary Energy (MeV)

MAT 562

# Fission Energy Distribution

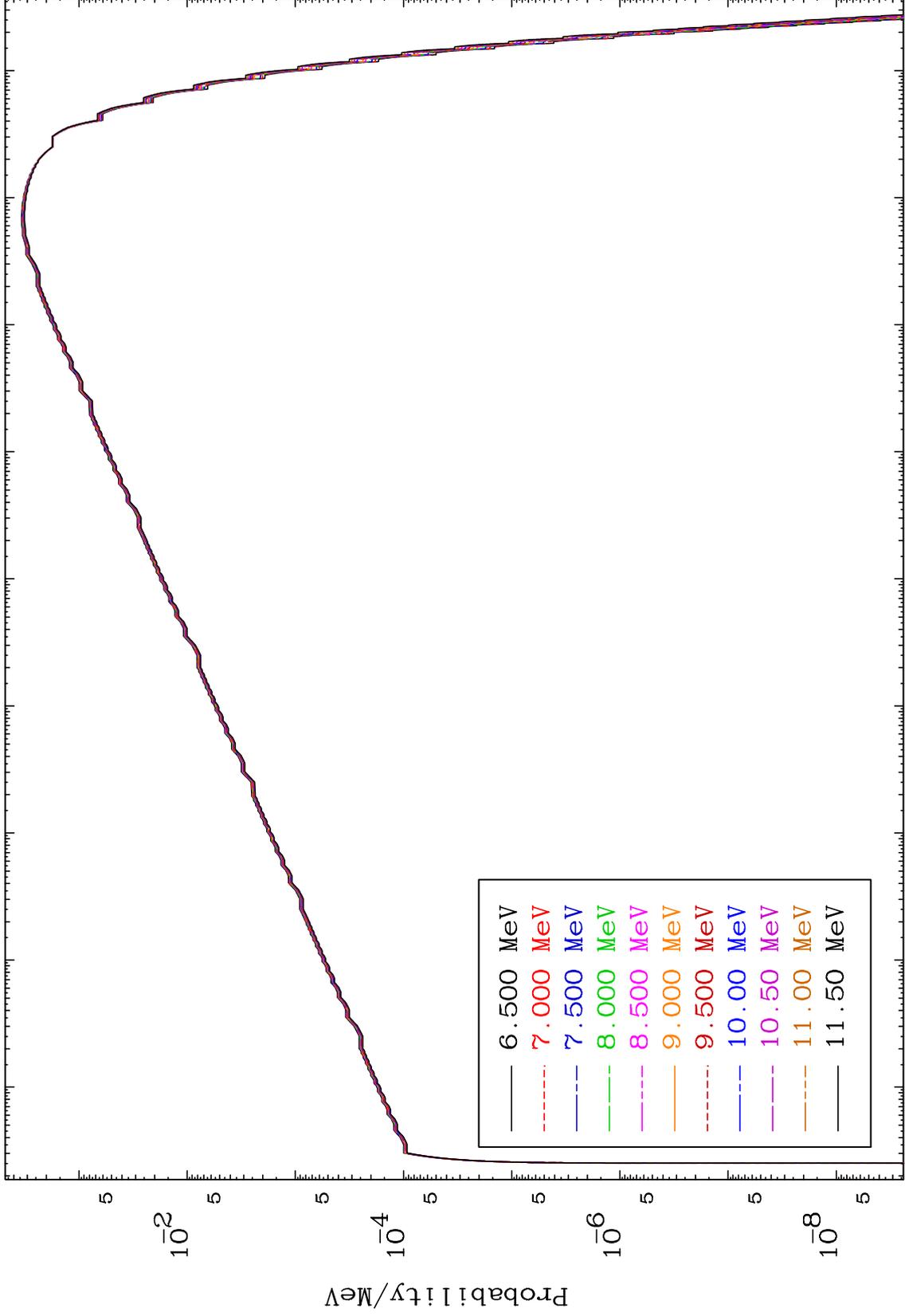
105-Db-262



MAT 562

### Fission Energy Distribution

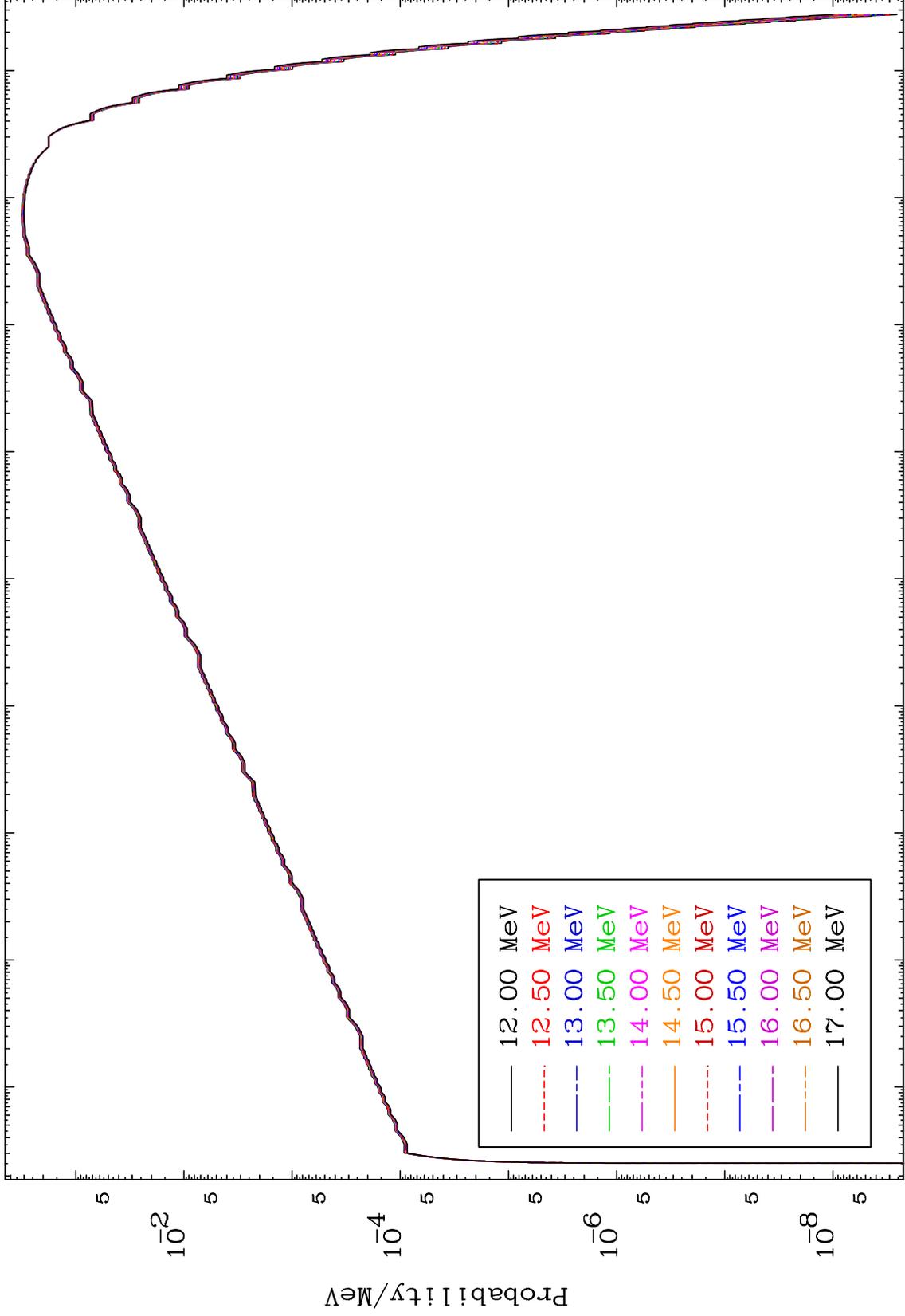
105-Db-262



MAT 562

# Fission Energy Distribution

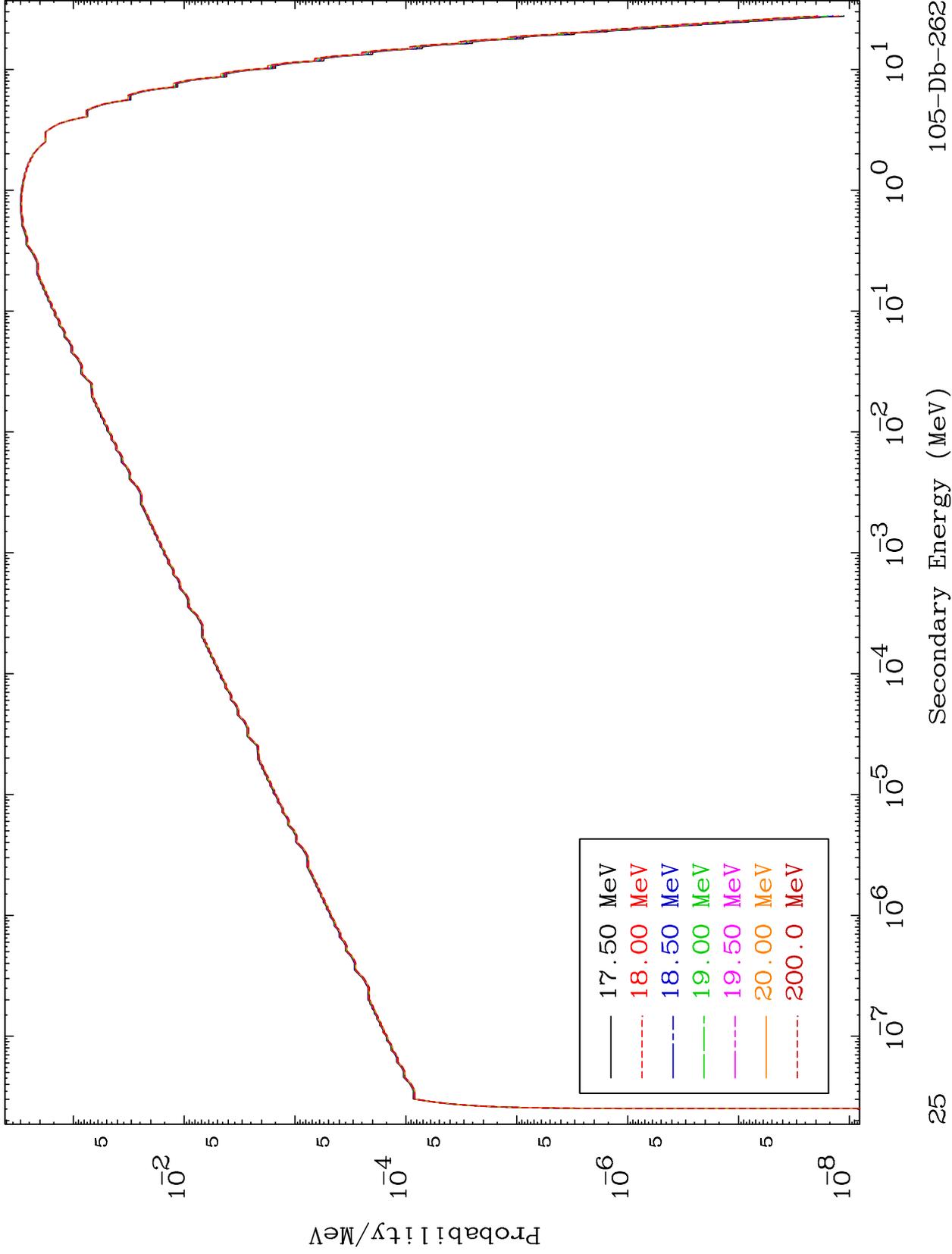
105-Db-262



MAT 562

# Fission Energy Distribution

105-Db-262

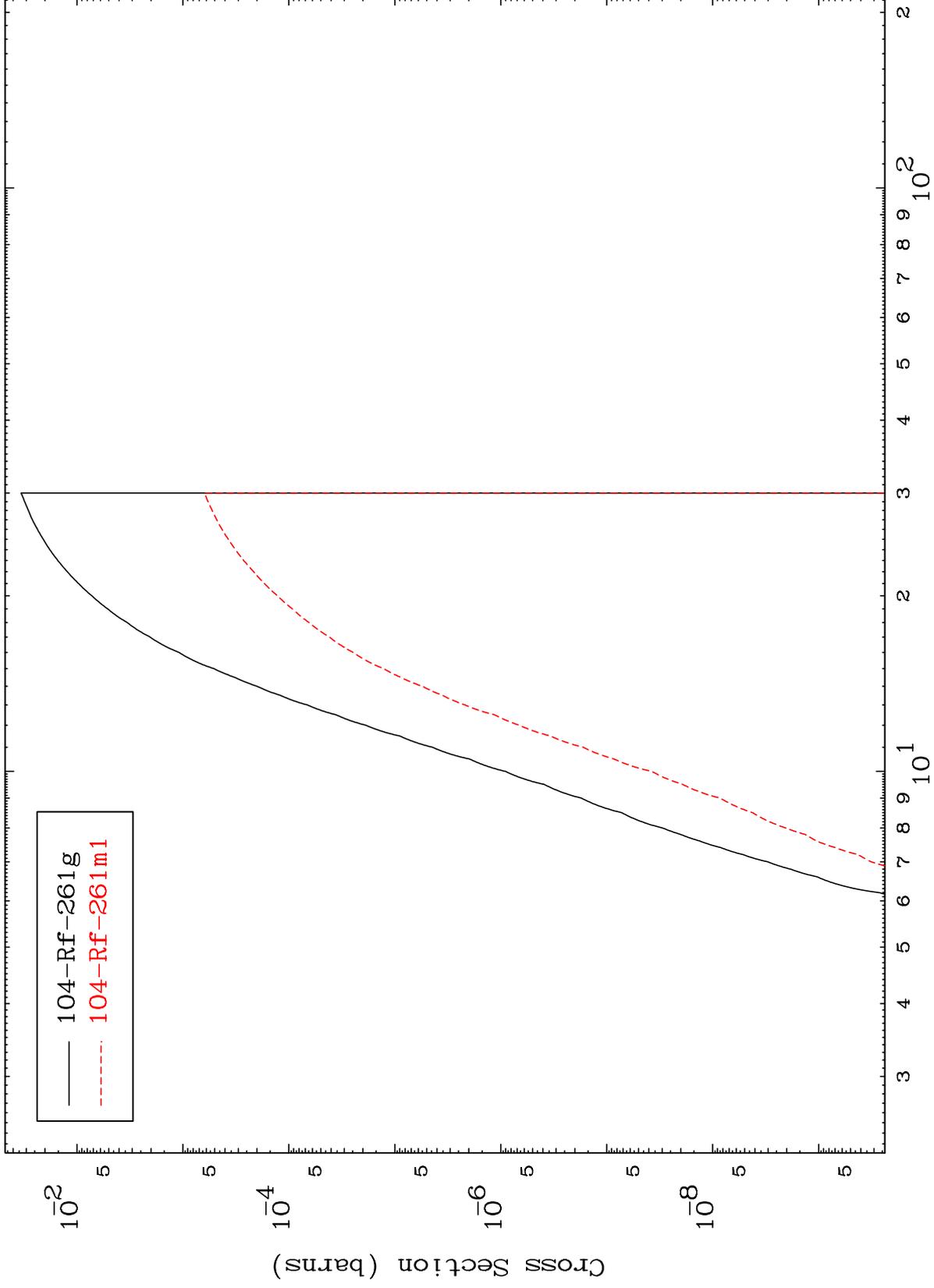


MAT 562

(n,n') p

105-Db-262

Radionuclide Production Cross Section



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Incident Energy (MeV)

105-Db-262

MAT 562

(n,n') 2 $\alpha$

105-Db-262

Radionuclide Production Cross Section

