

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

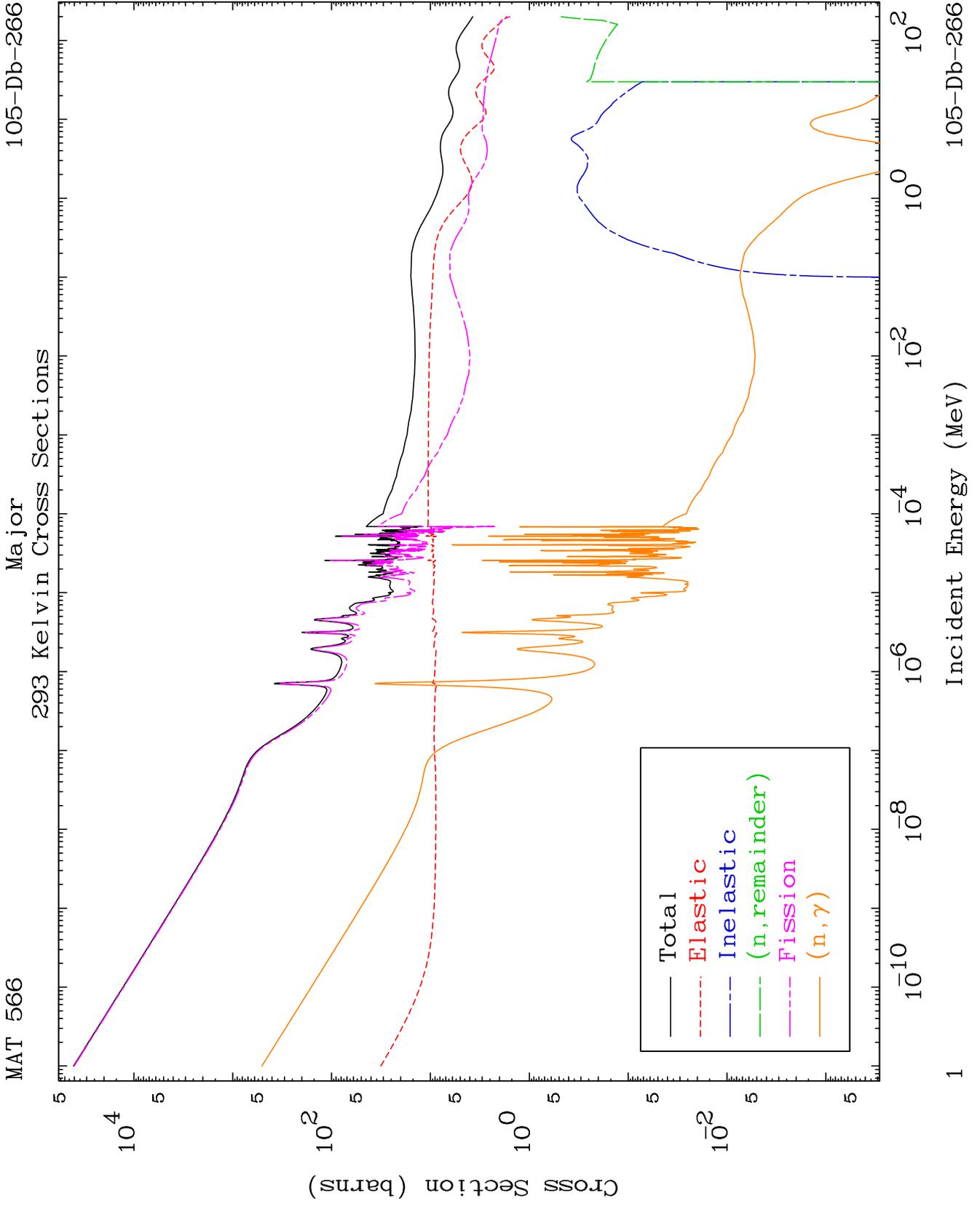
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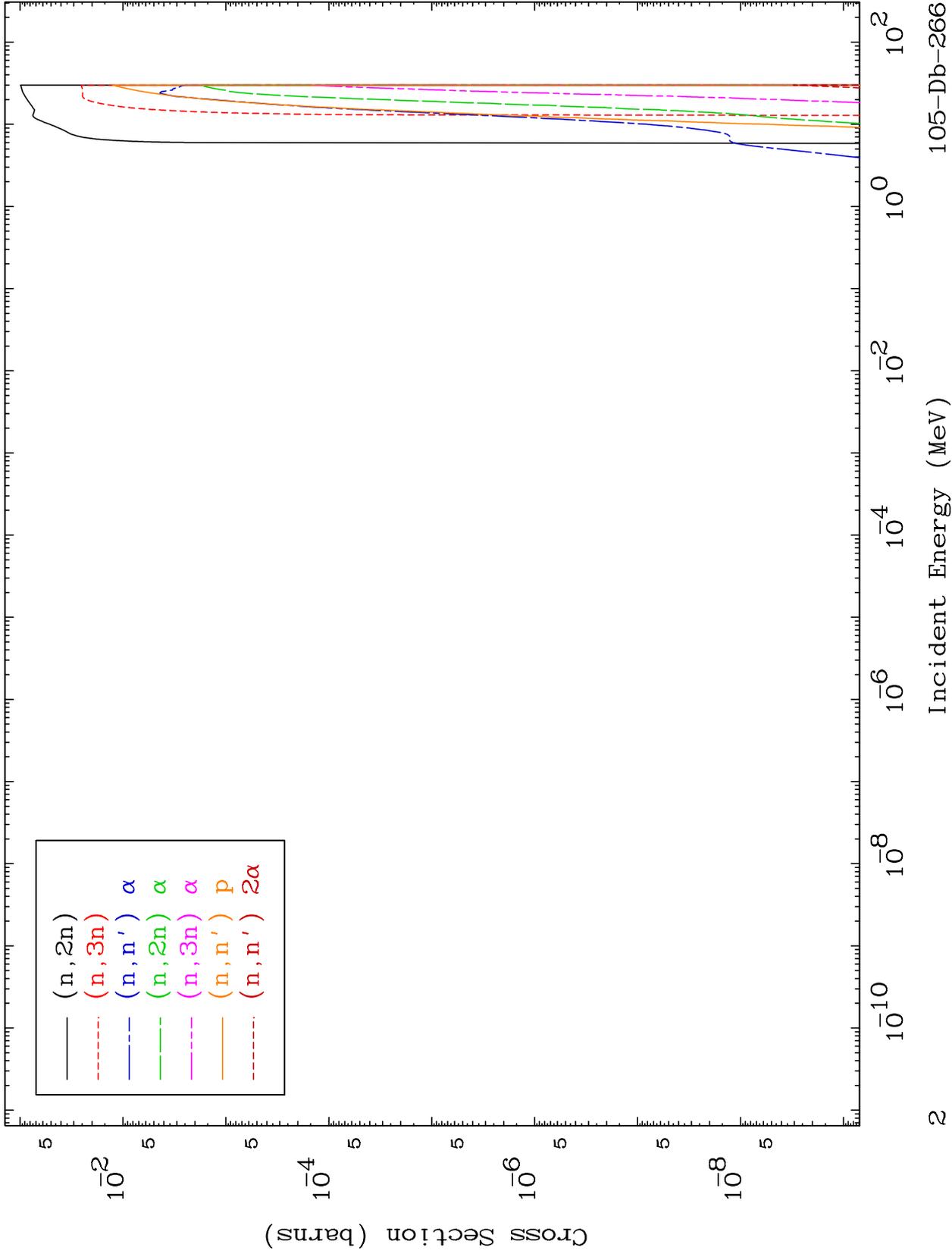
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MAT 566

Neutron Production
293 Kelvin Cross Sections

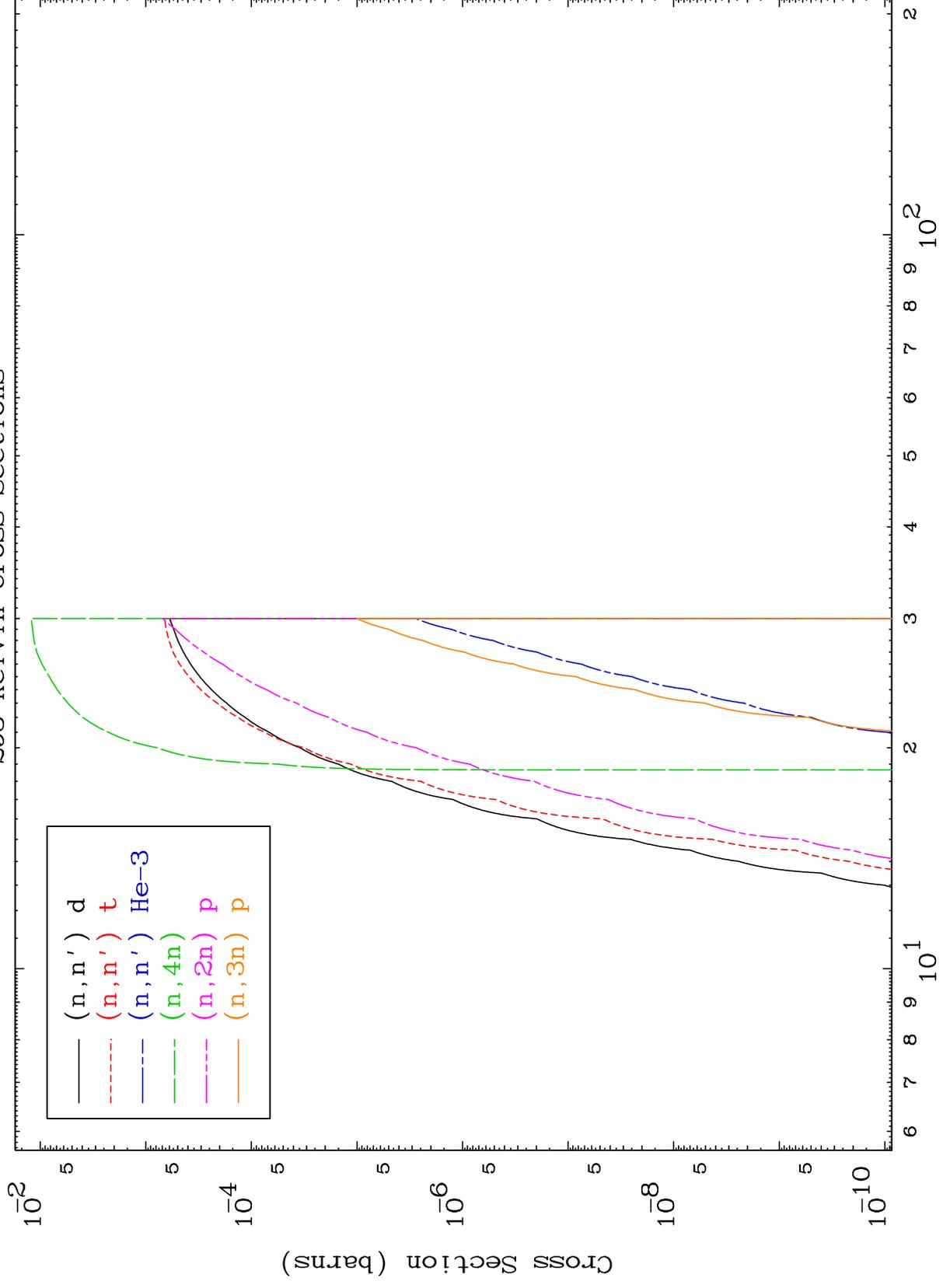
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MAT 566

Neutron Production
293 Kelvin Cross Sections

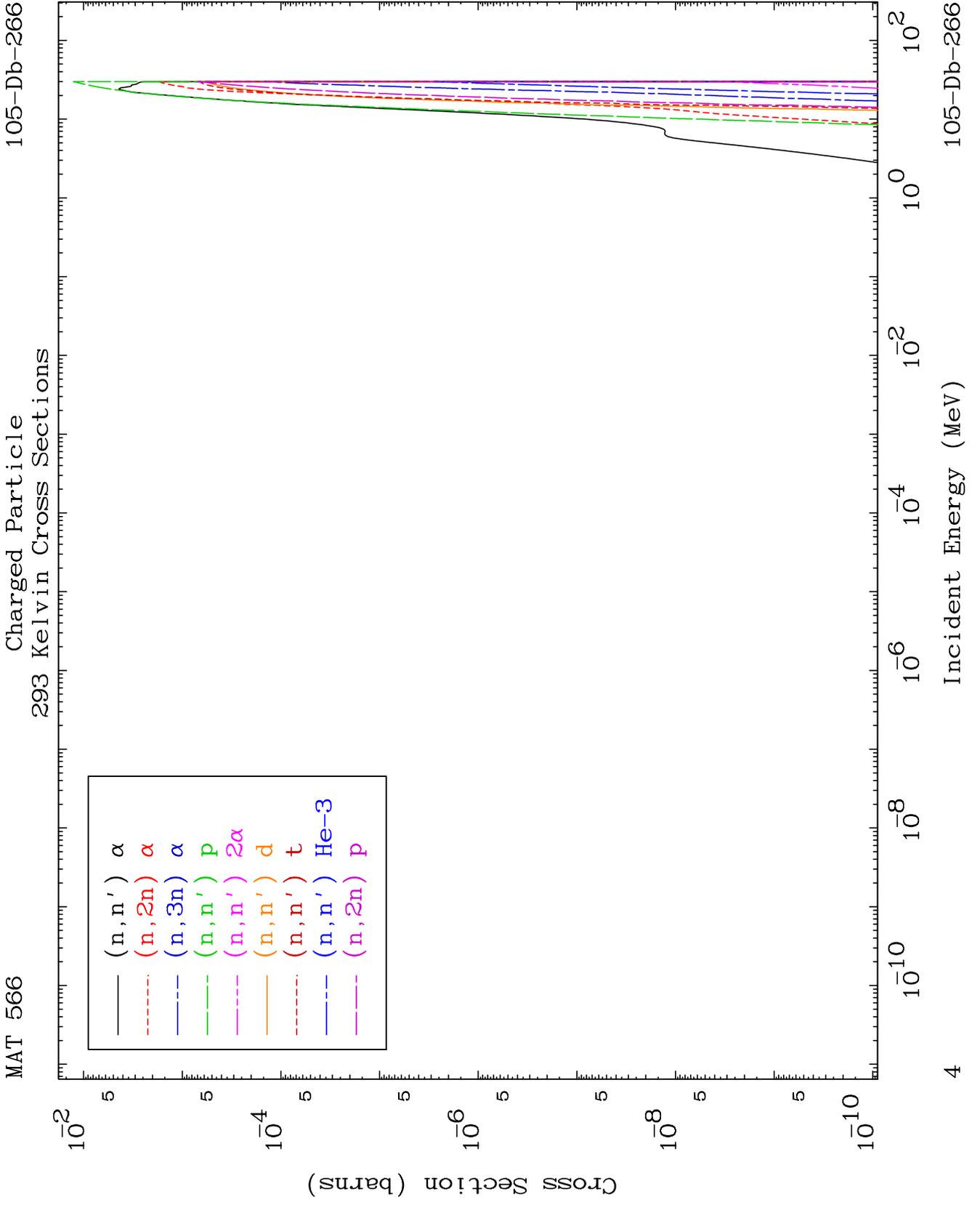
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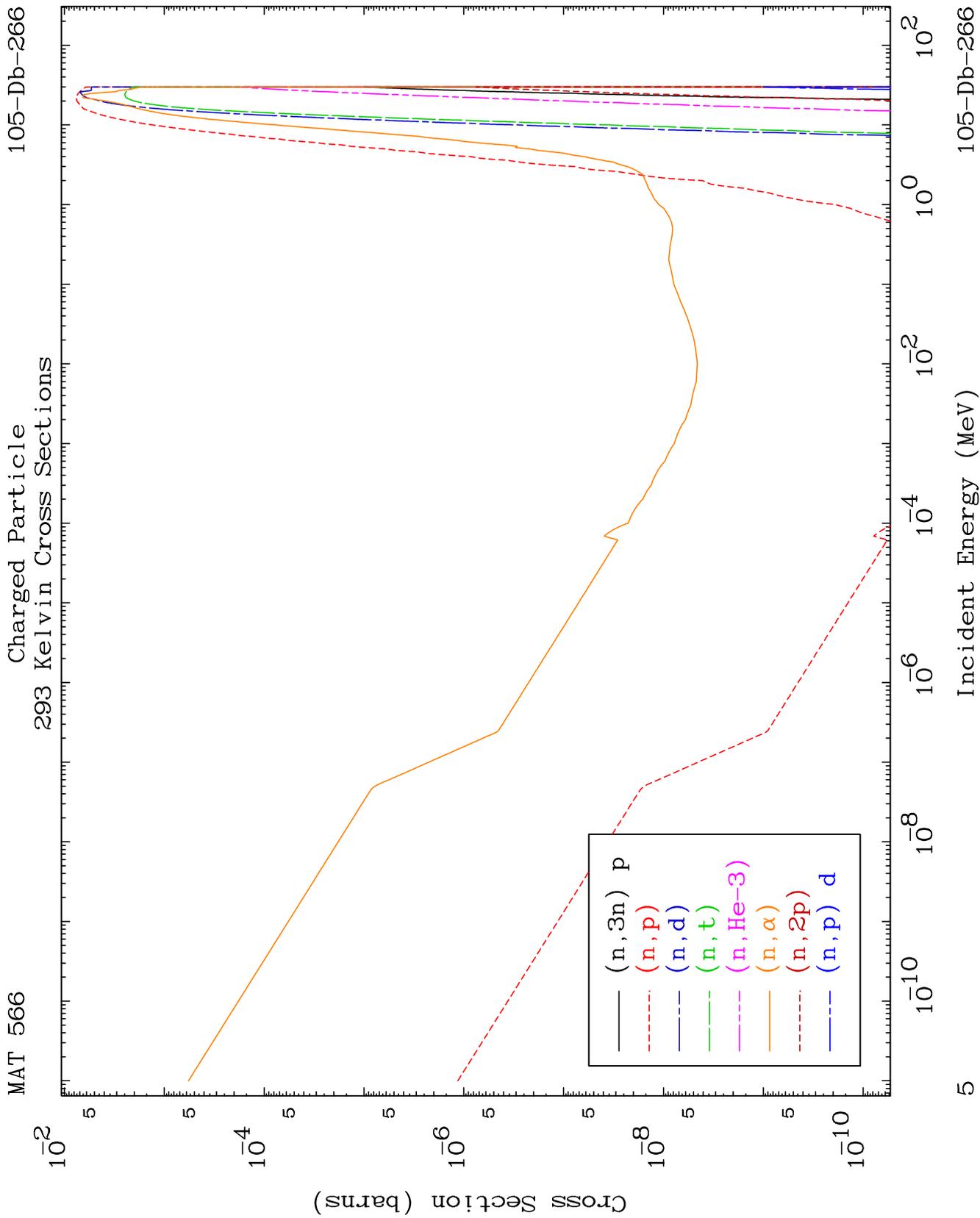


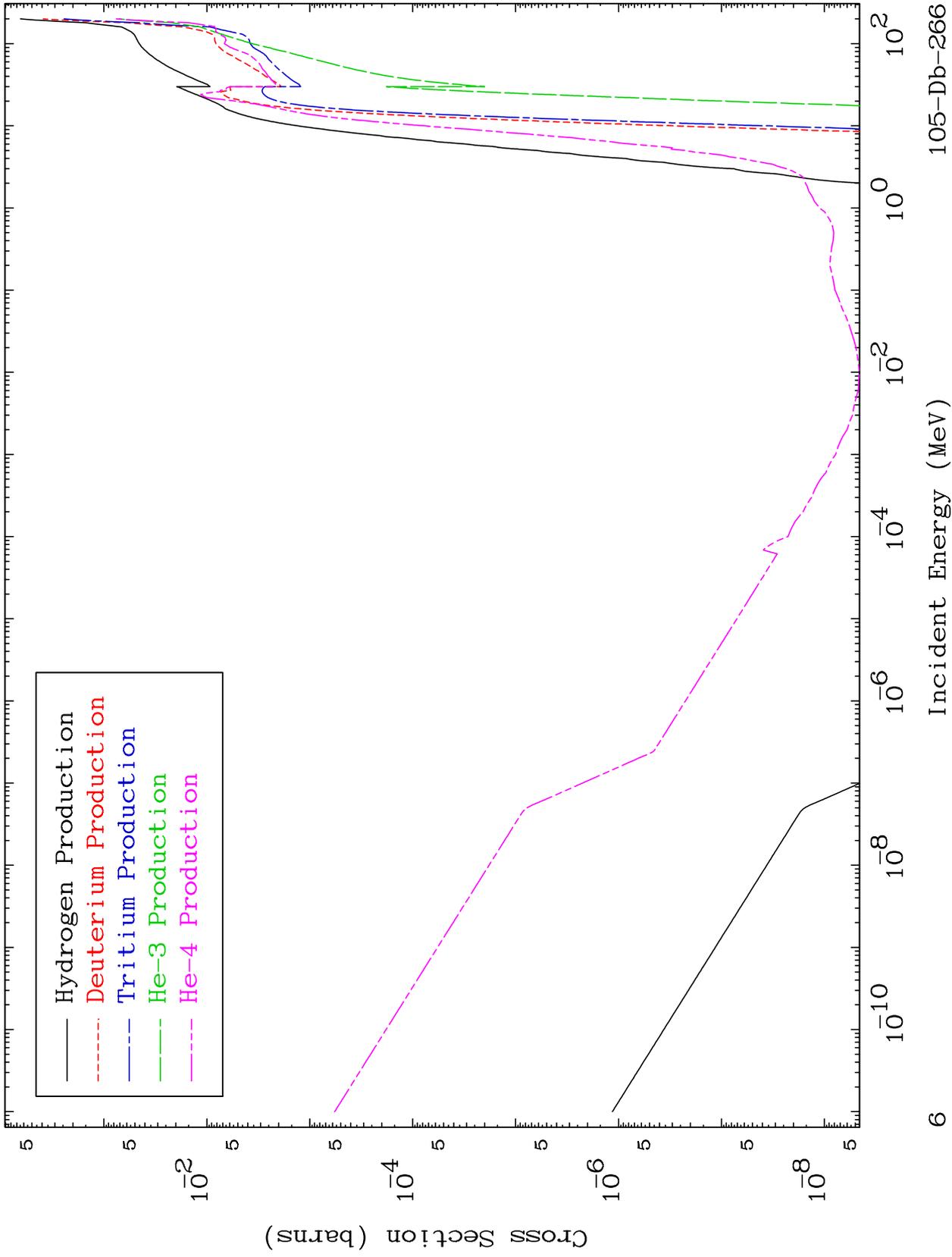
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Incident Energy (MeV)

105-Db-266



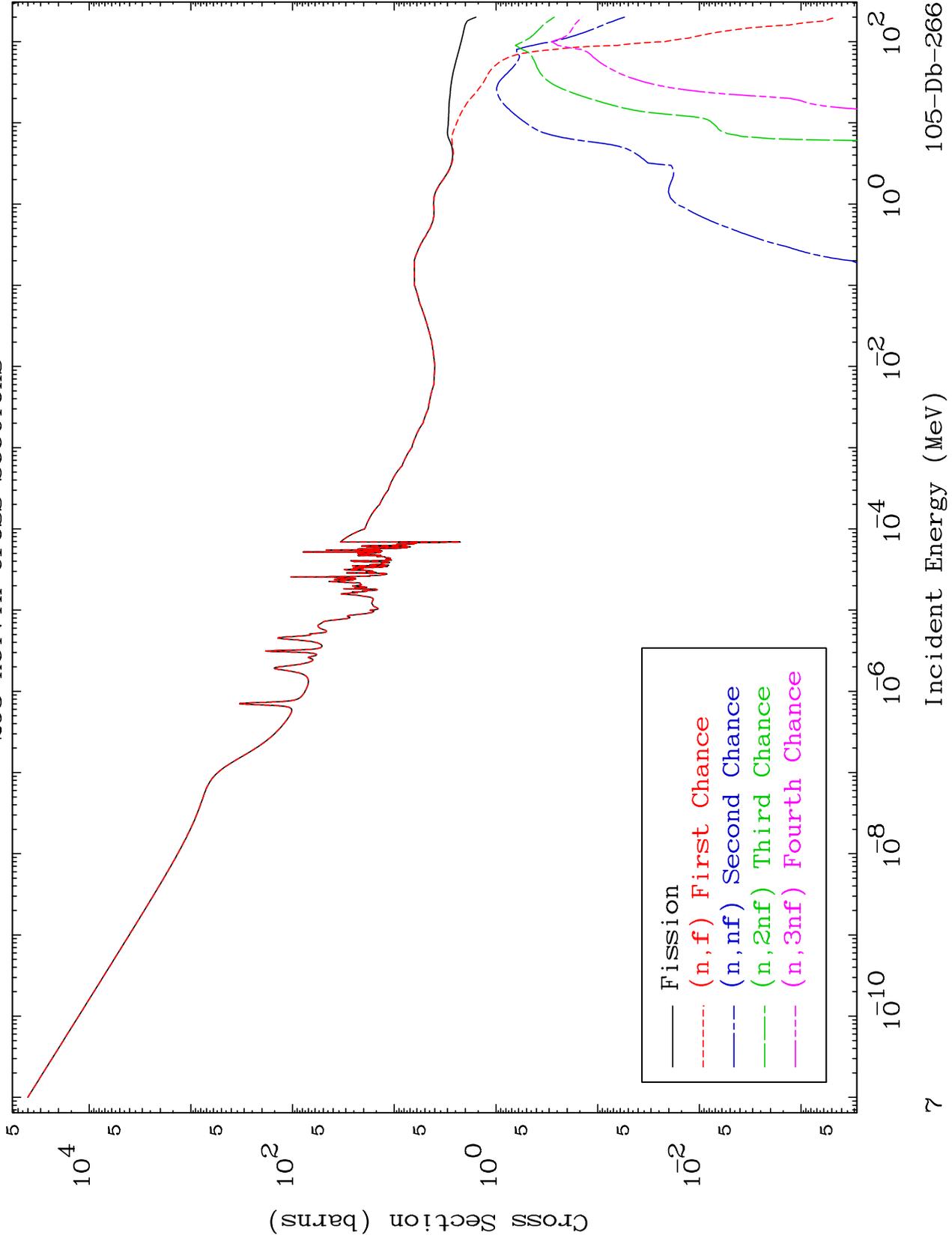




MAT 566

Fission
293 Kelvin Cross Sections

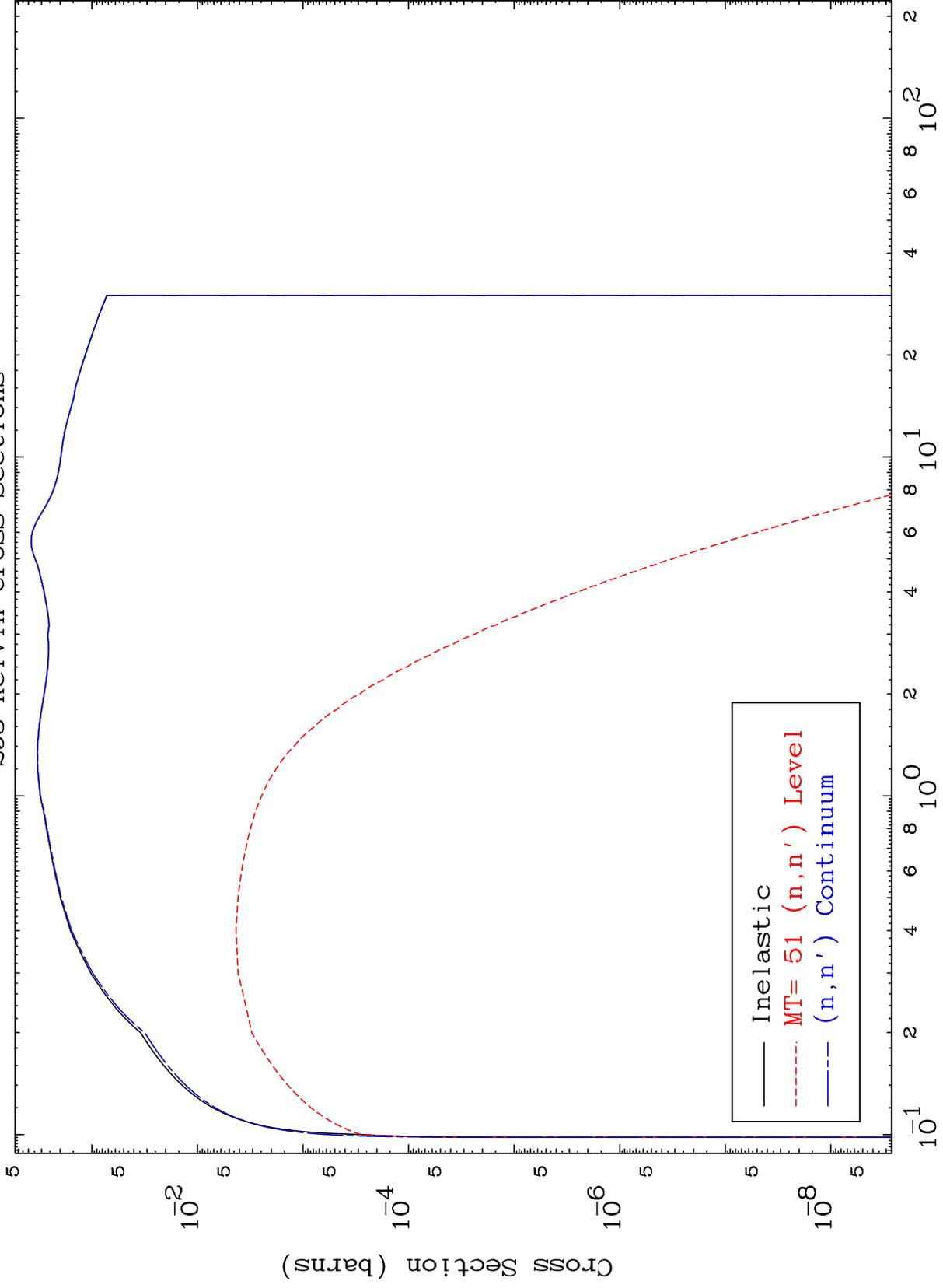
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MAT 566

(n,n') Level
293 Kelvin Cross Sections

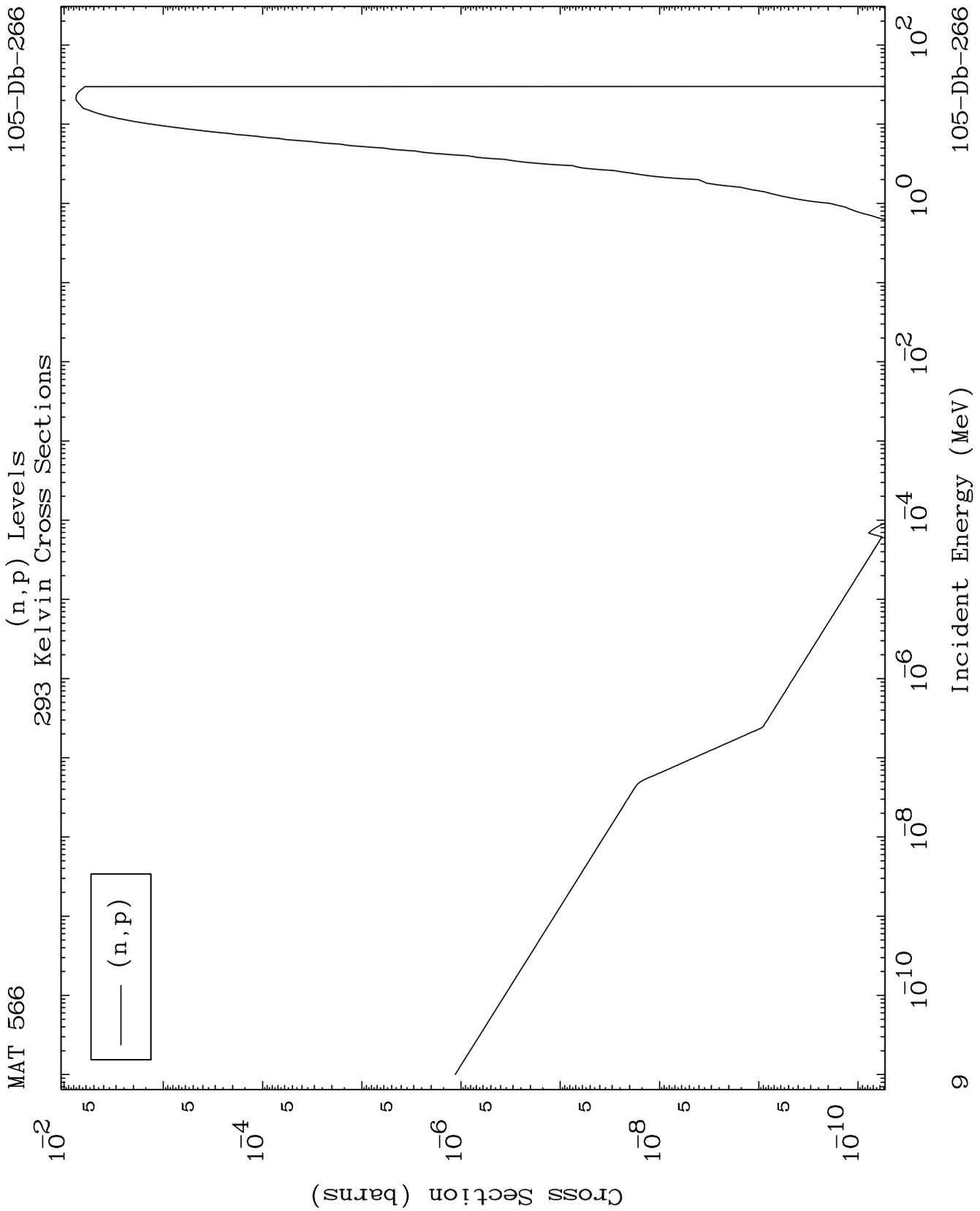
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Incident Energy (MeV)

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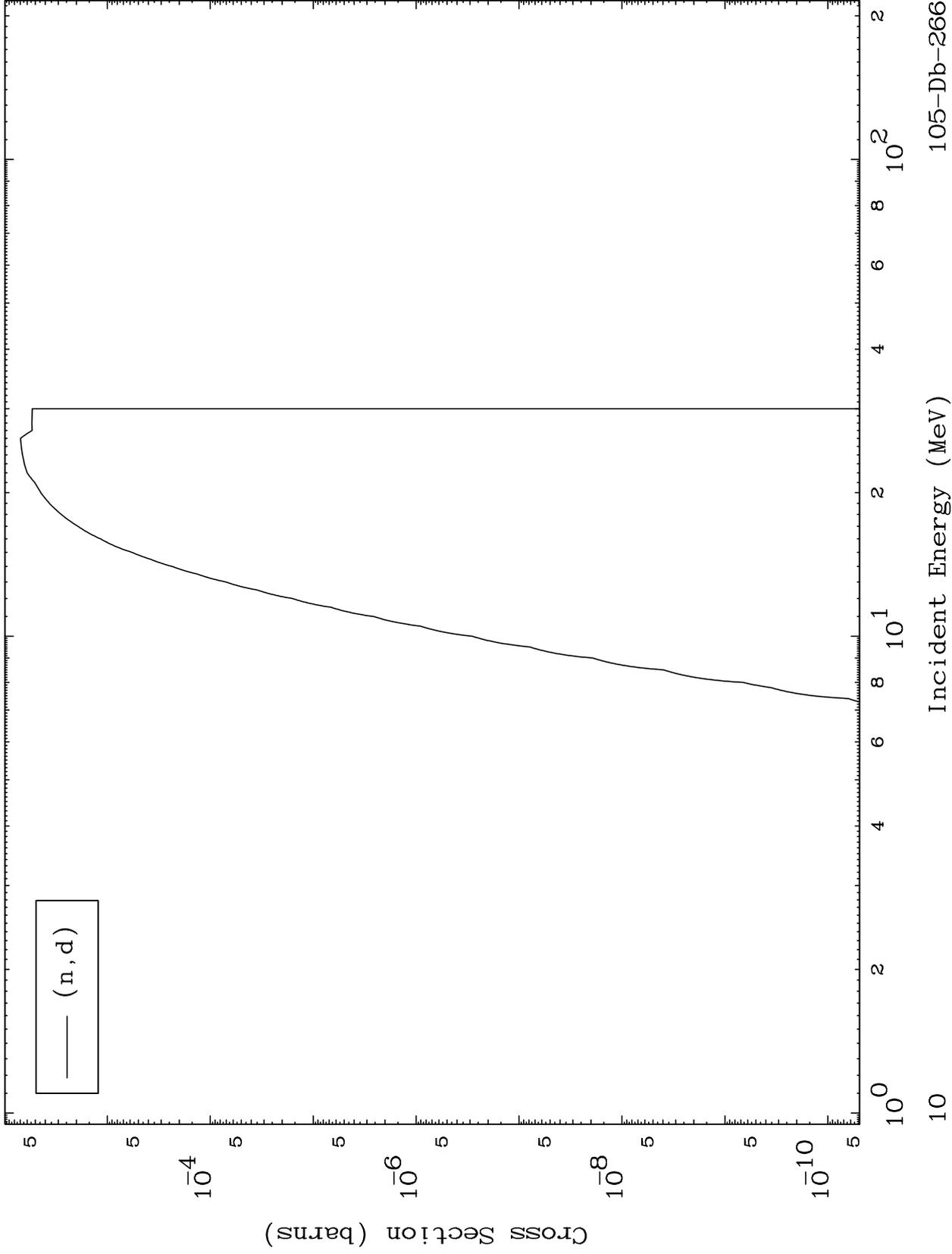
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MAT 566

(n,d) Levels
293 Kelvin Cross Sections

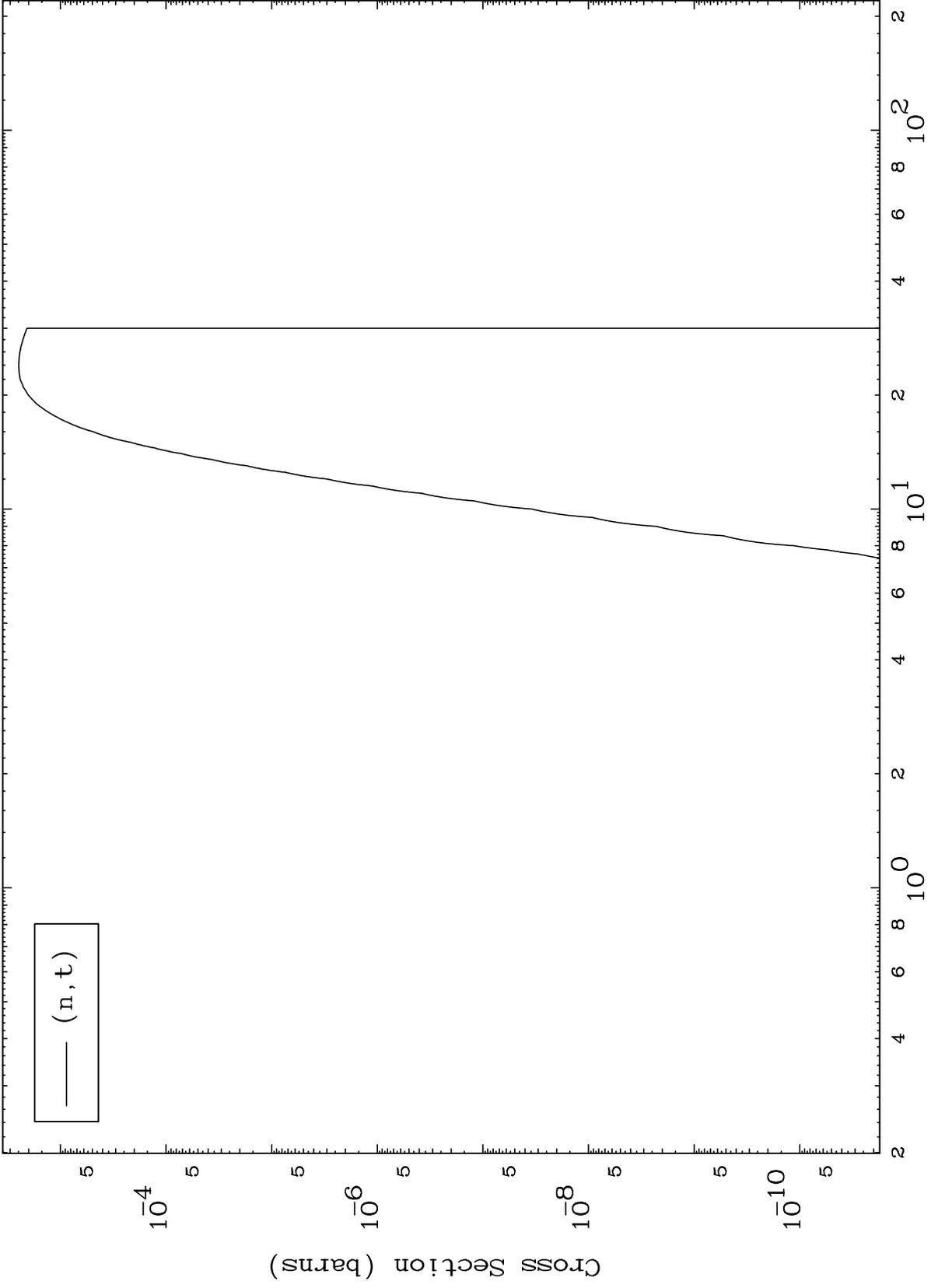
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(n,t) Levels
293 Kelvin Cross Sections

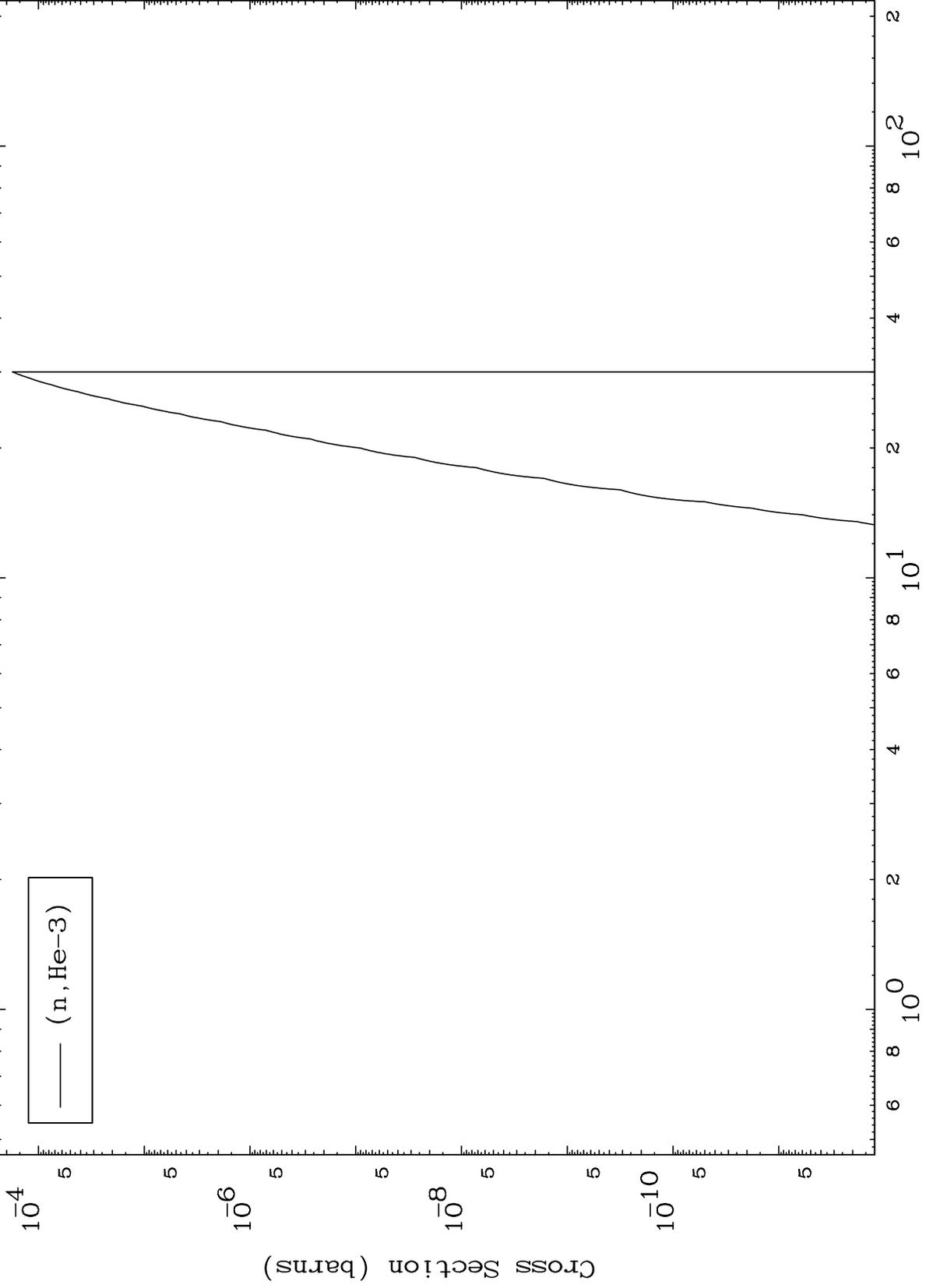
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(n,He3) Levels
293 Kelvin Cross Sections

105-Db-266



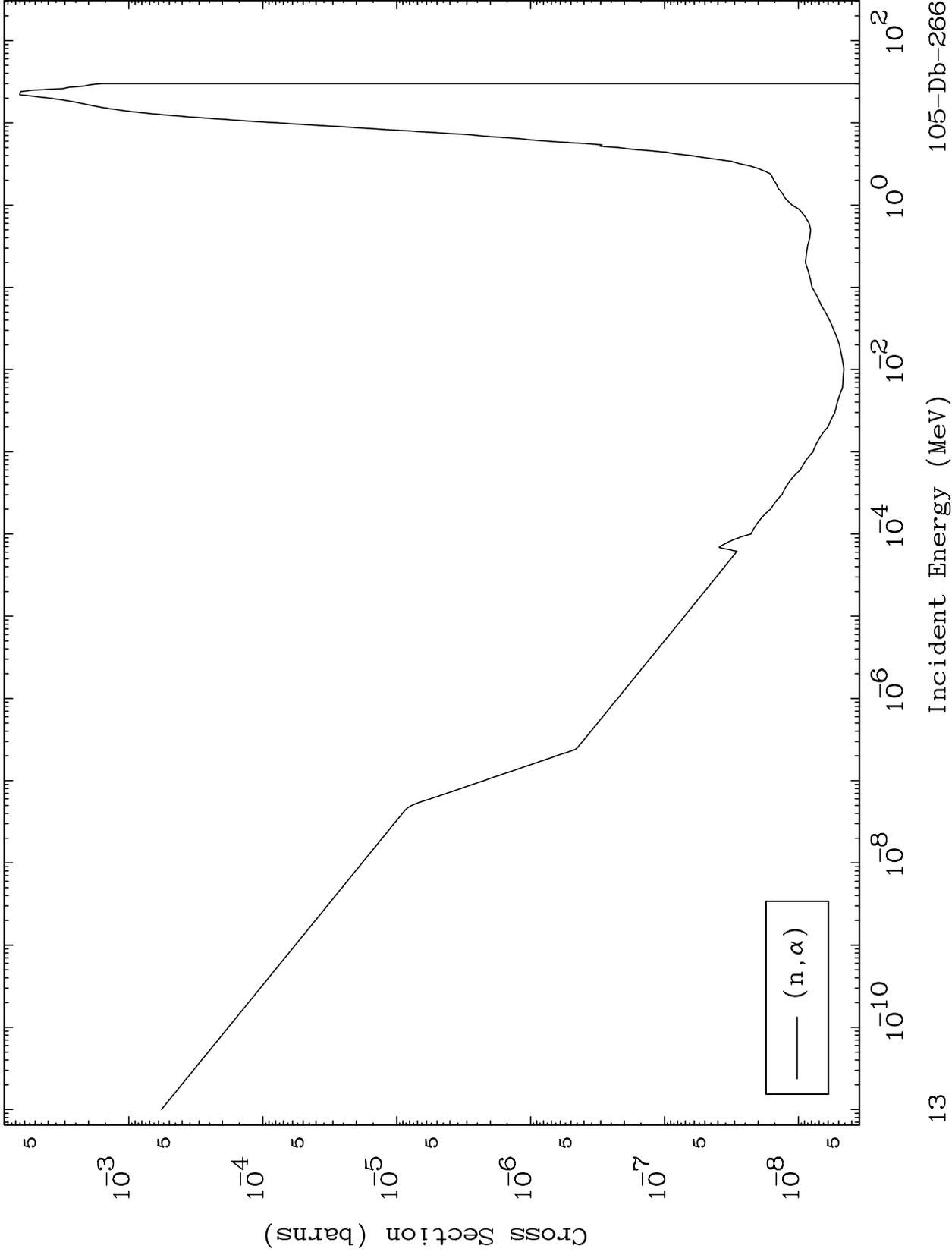
12

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(n, α) Levels
293 Kelvin Cross Sections

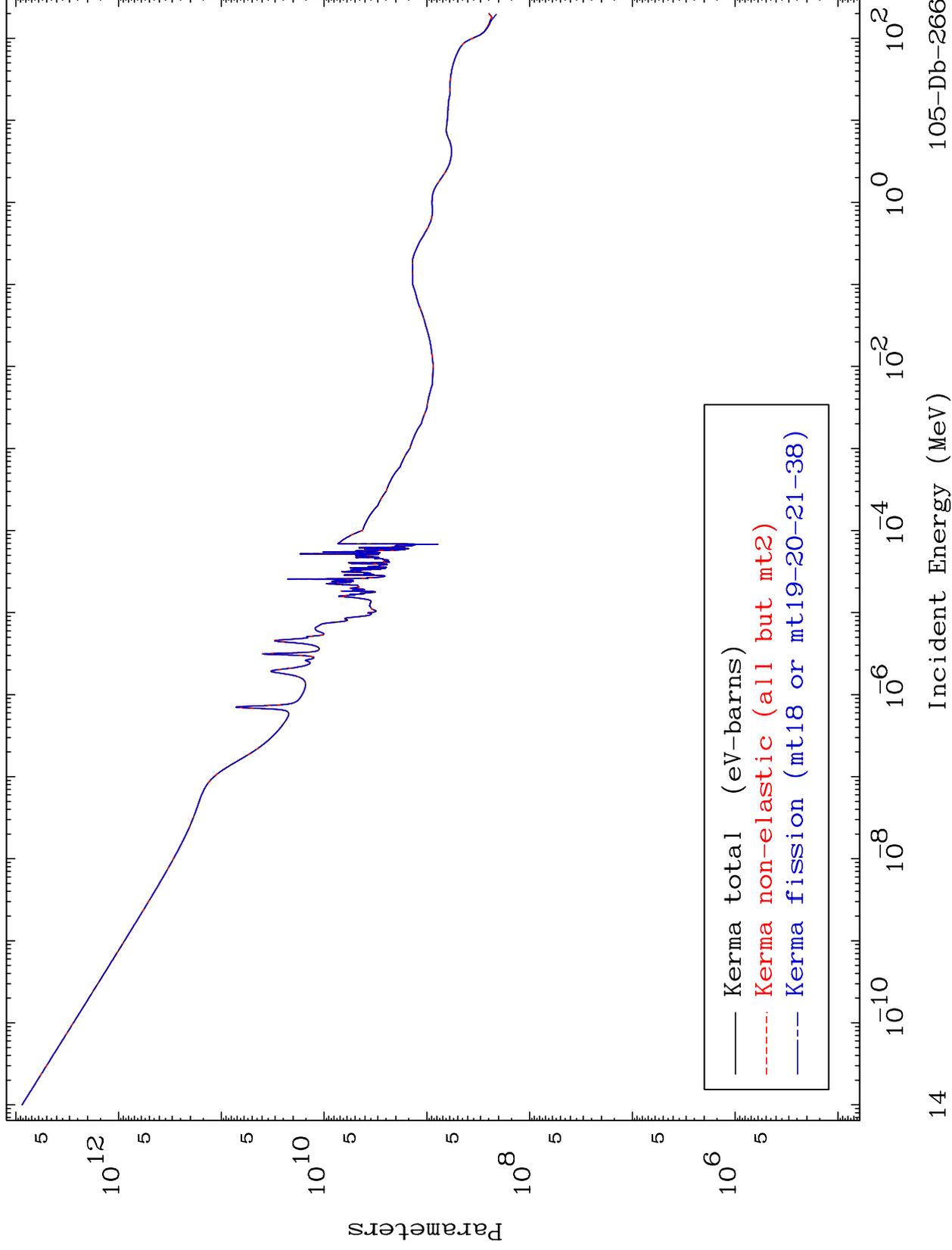
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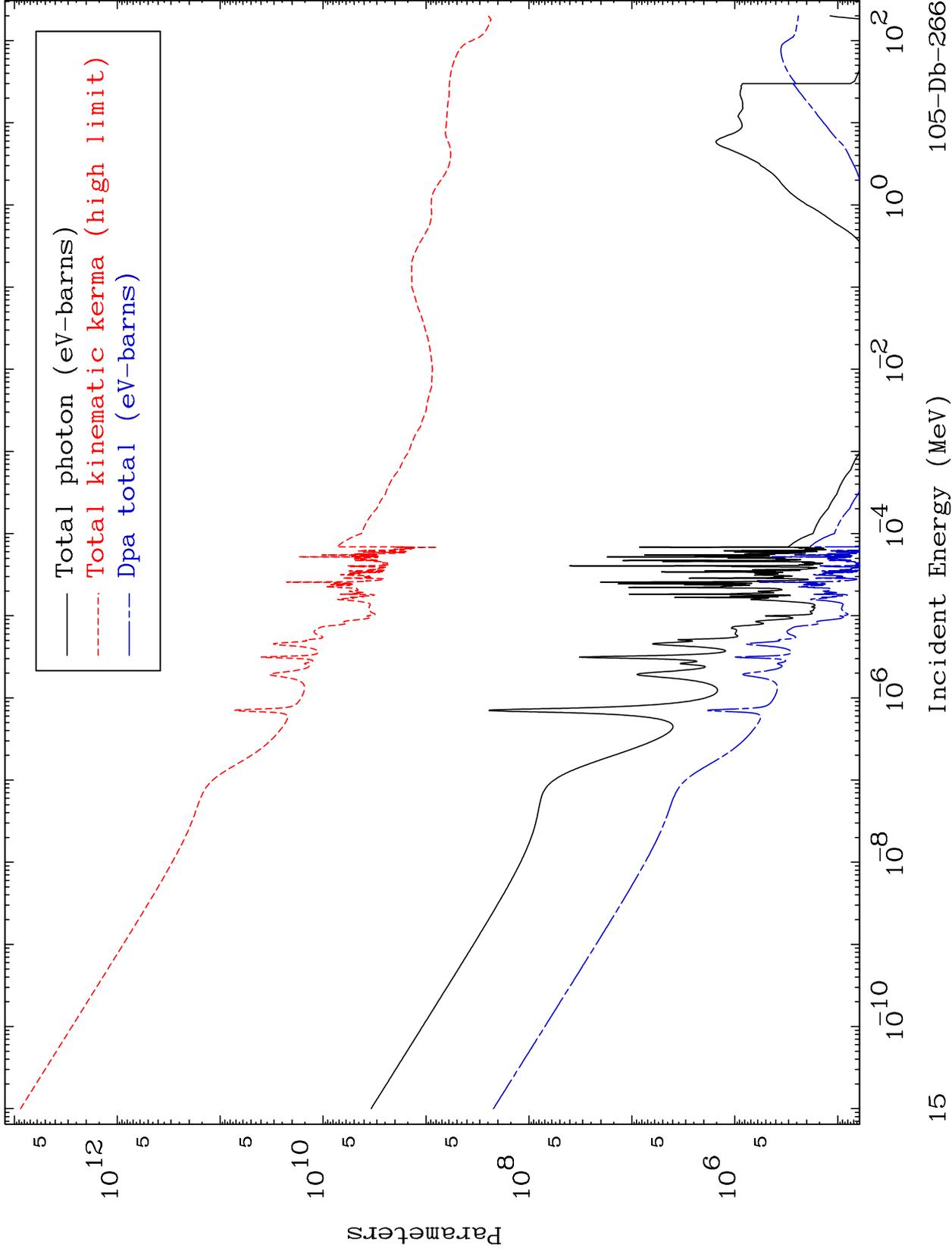


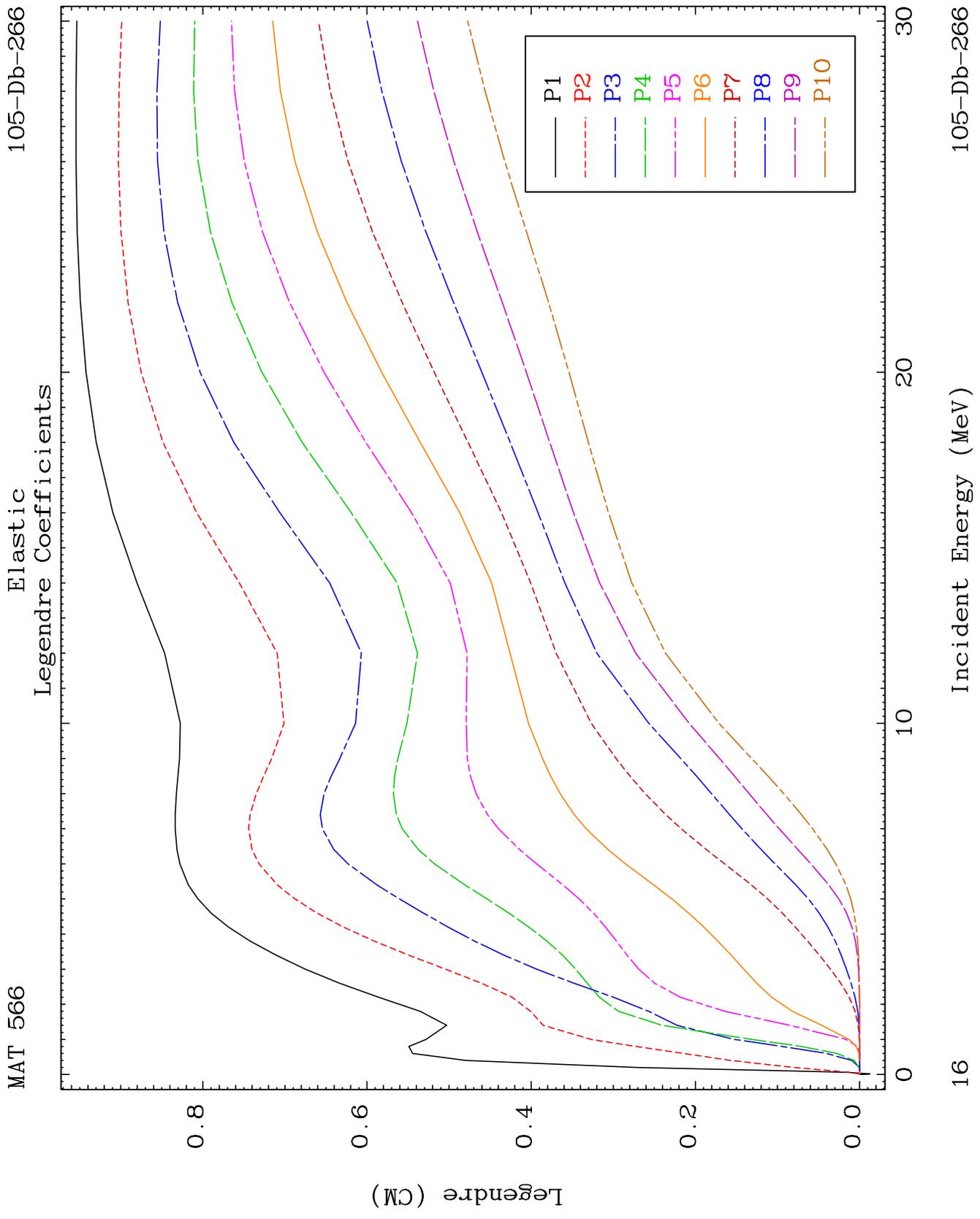
MAT 566

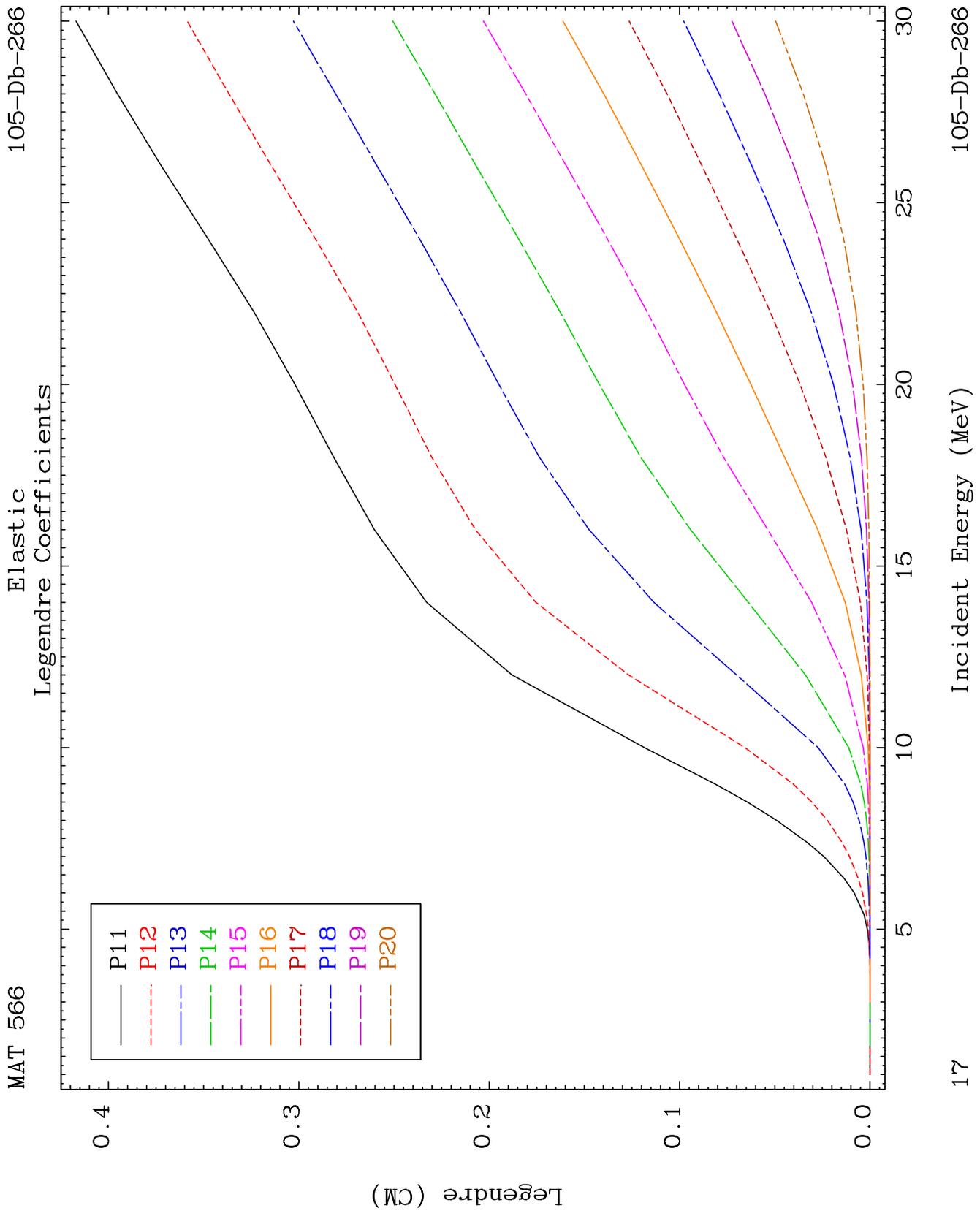
Energy Release
Parameters

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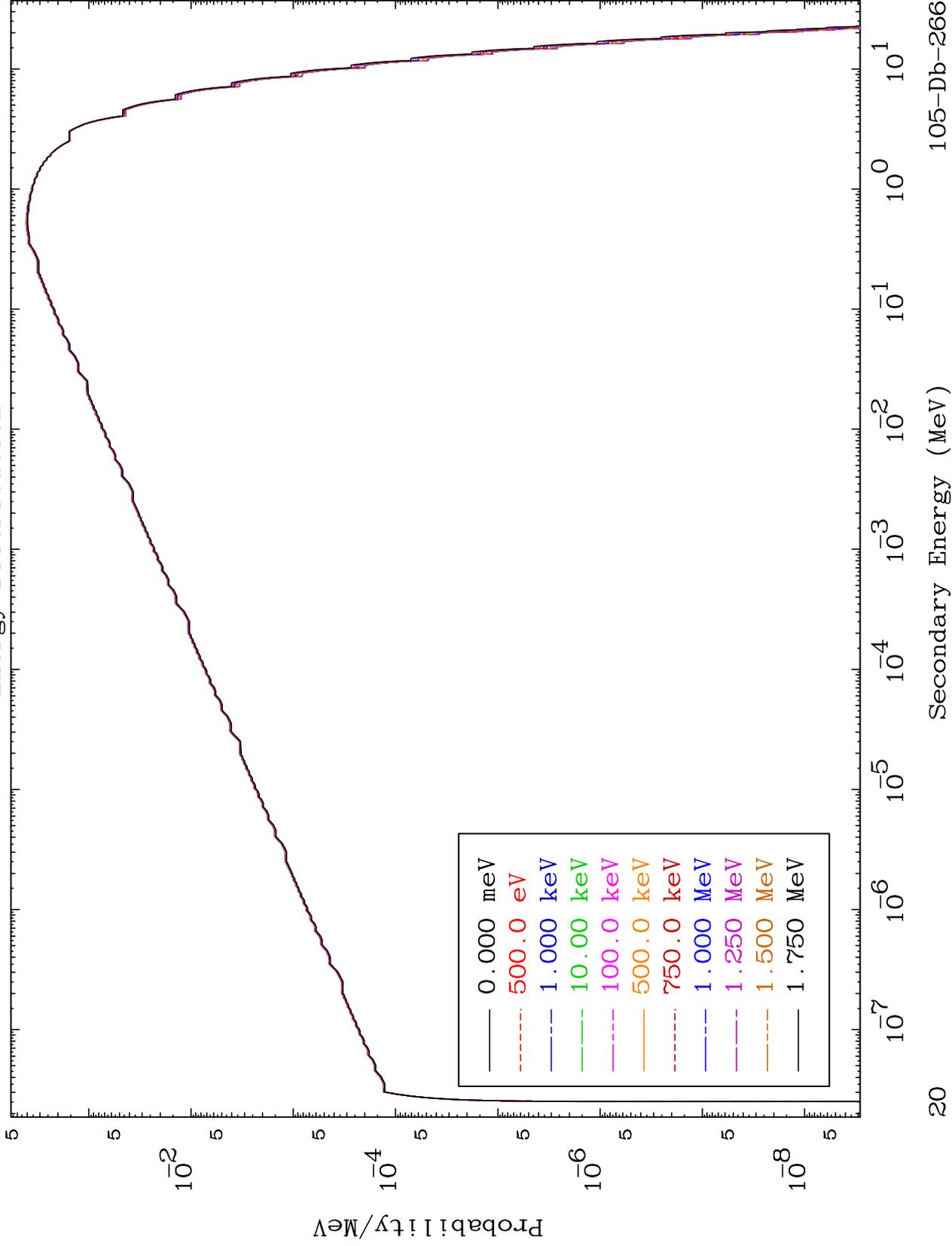




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Fission Energy Distribution

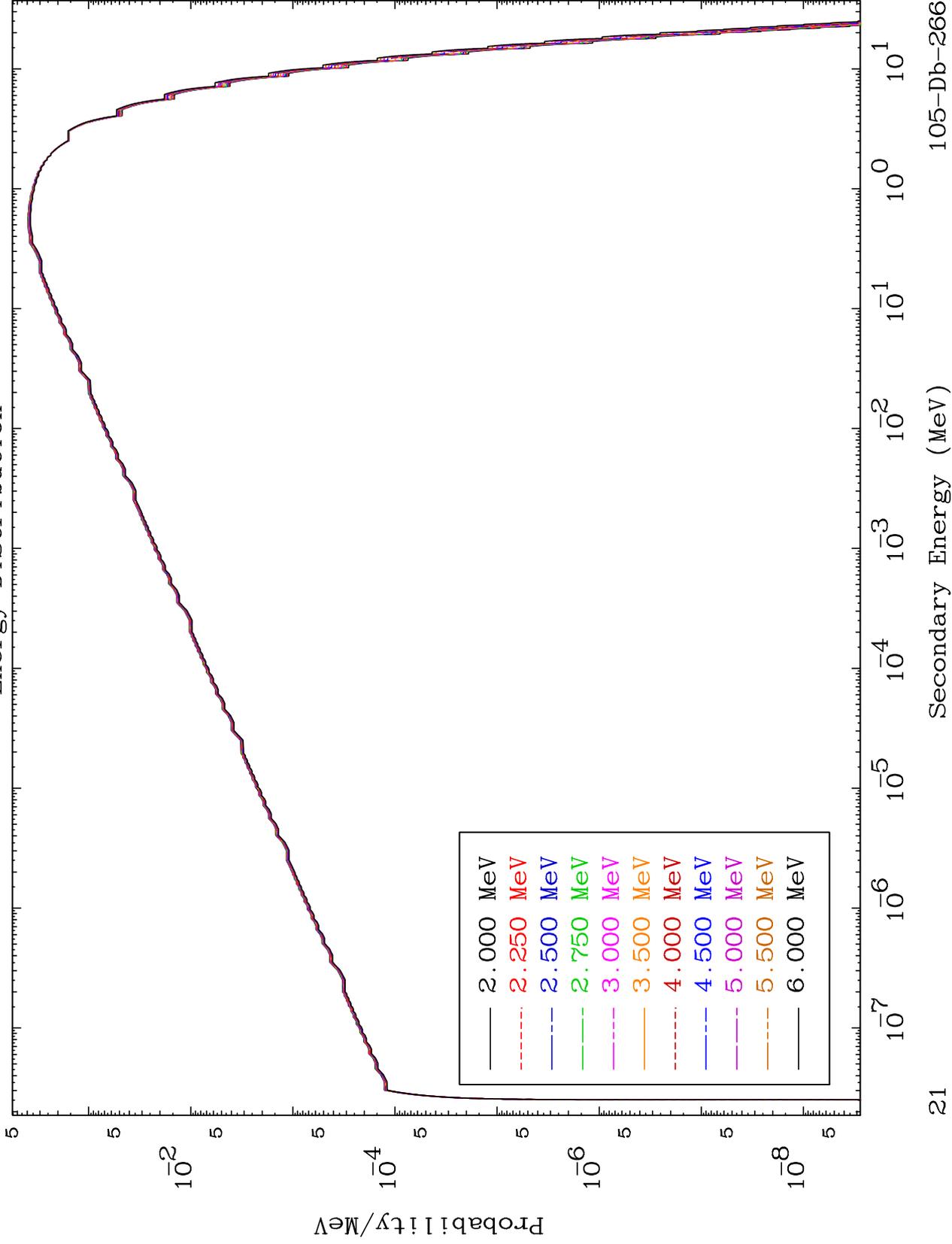
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Fission Energy Distribution

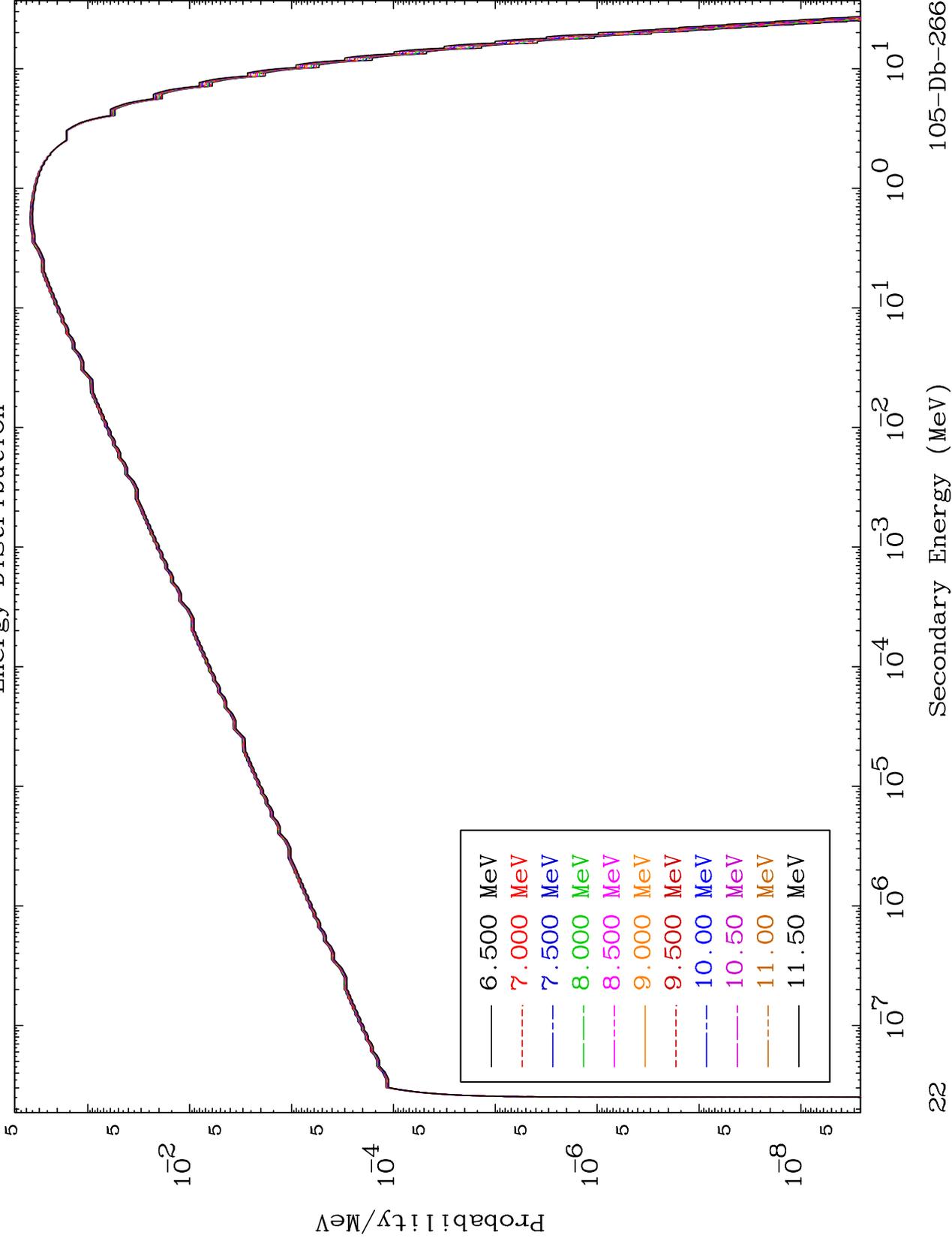
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Fission Energy Distribution

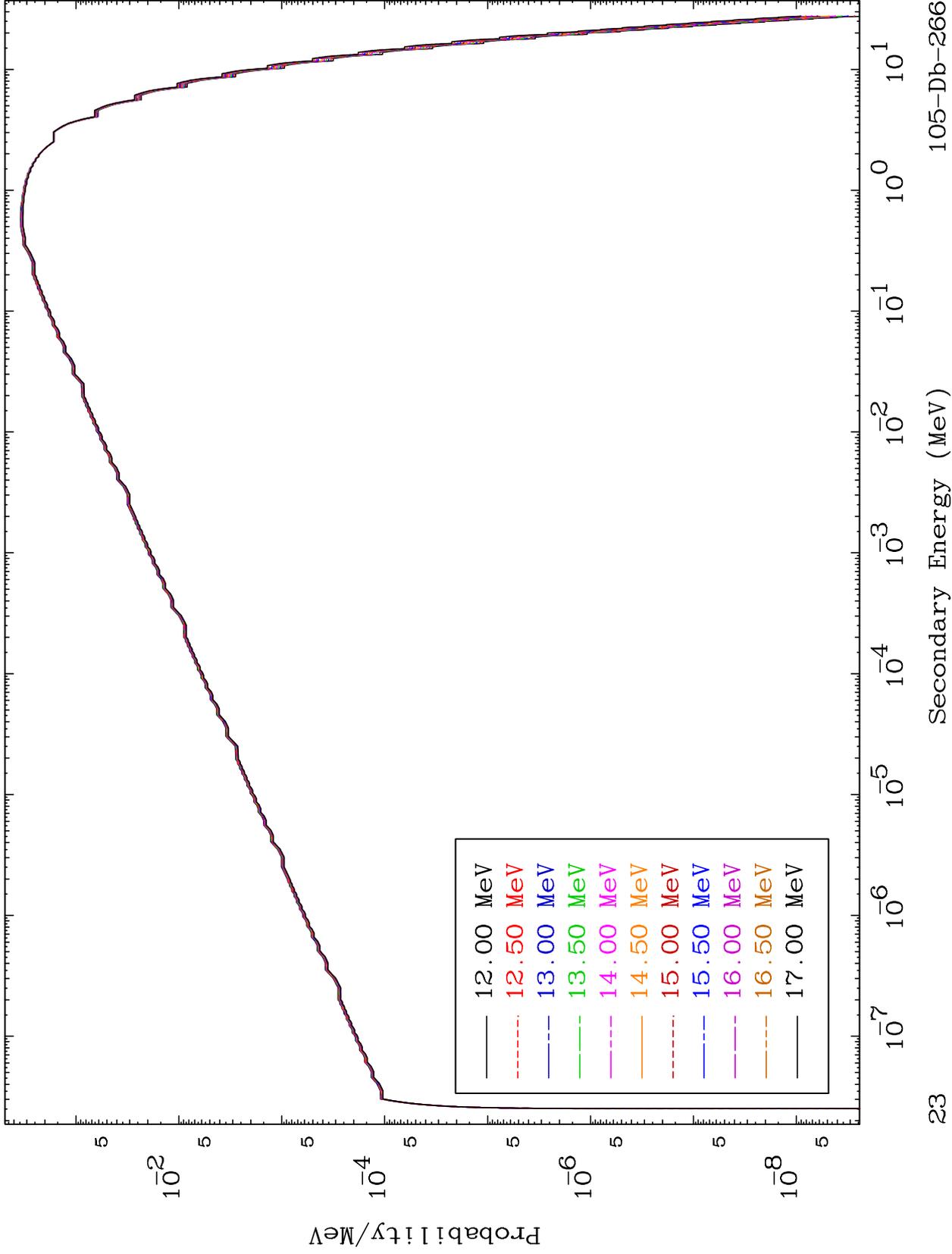
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Fission Energy Distribution

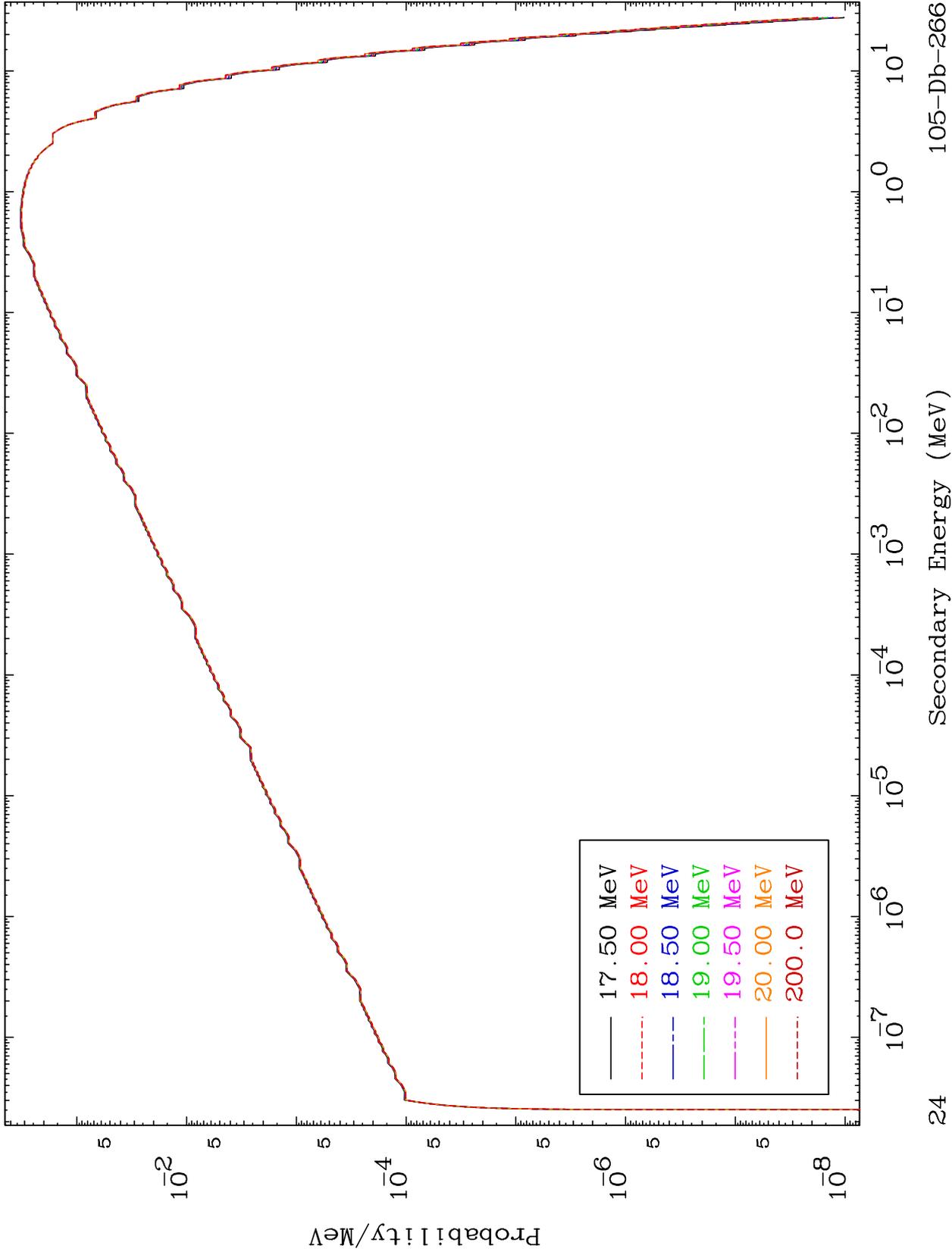
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Fission Energy Distribution

105-Db-266

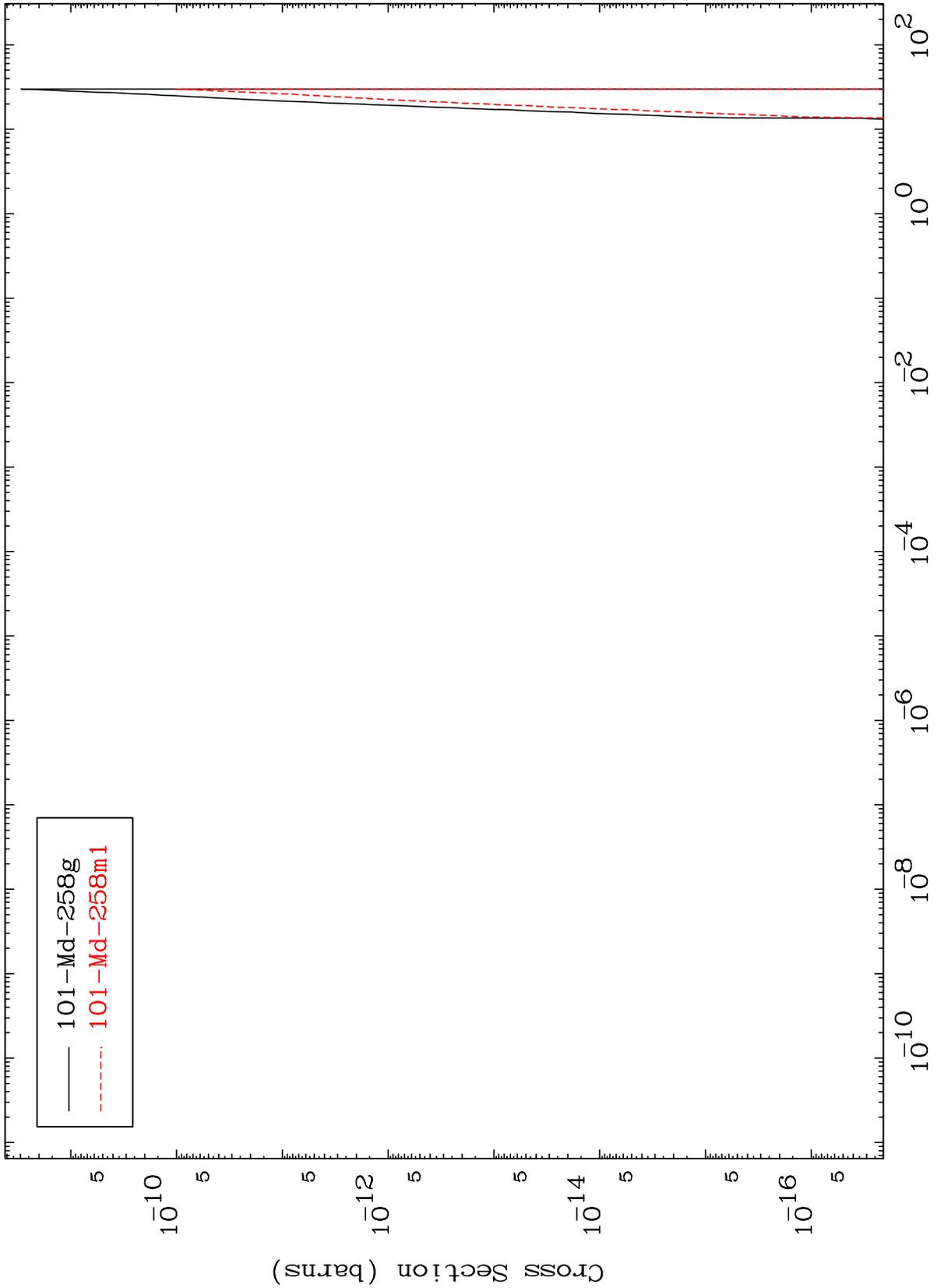


MAT 566

(n,n') 2 α

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Radionuclide Production Cross Section



25

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