

Program EVALPLOT
(Version 2017-1)

by

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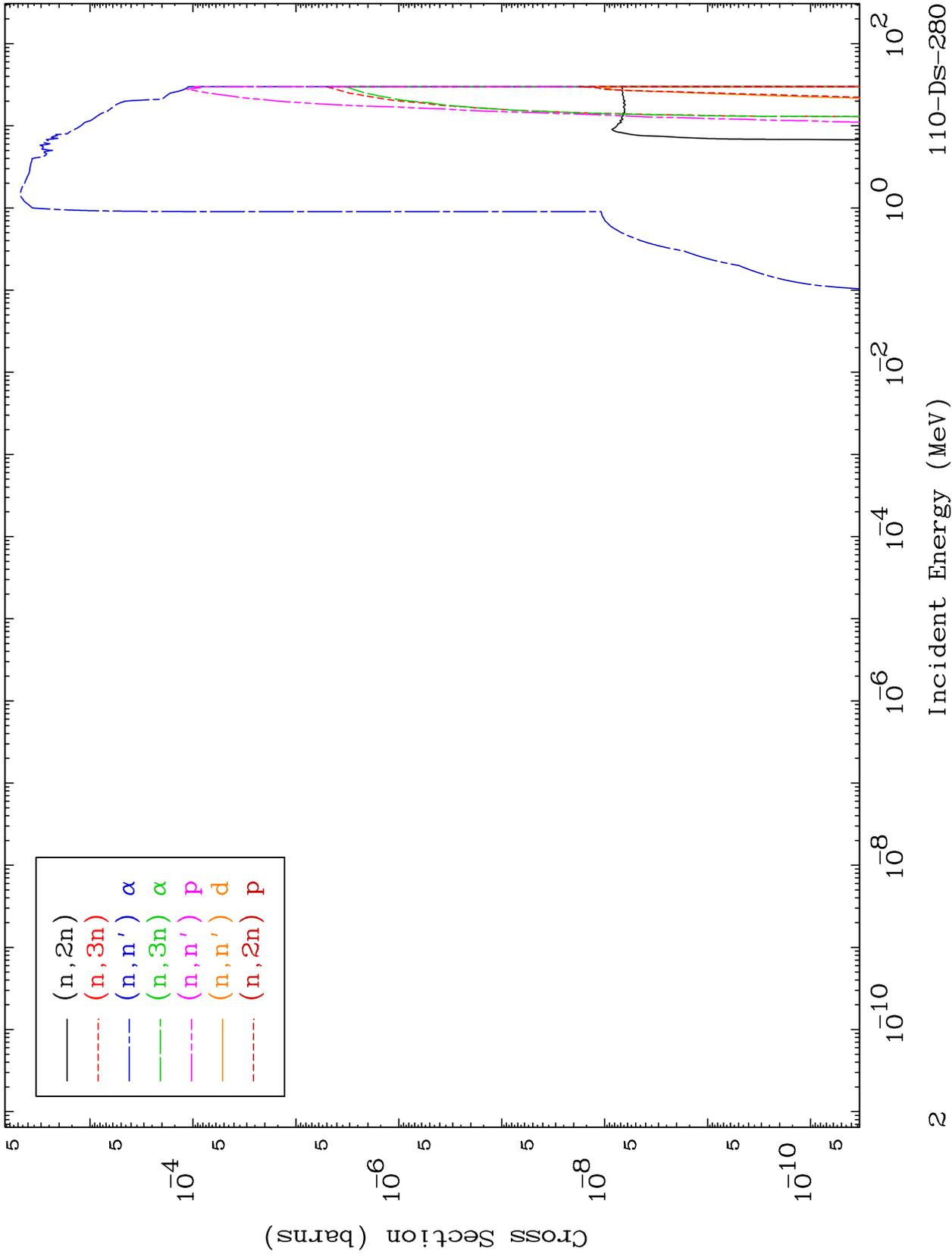
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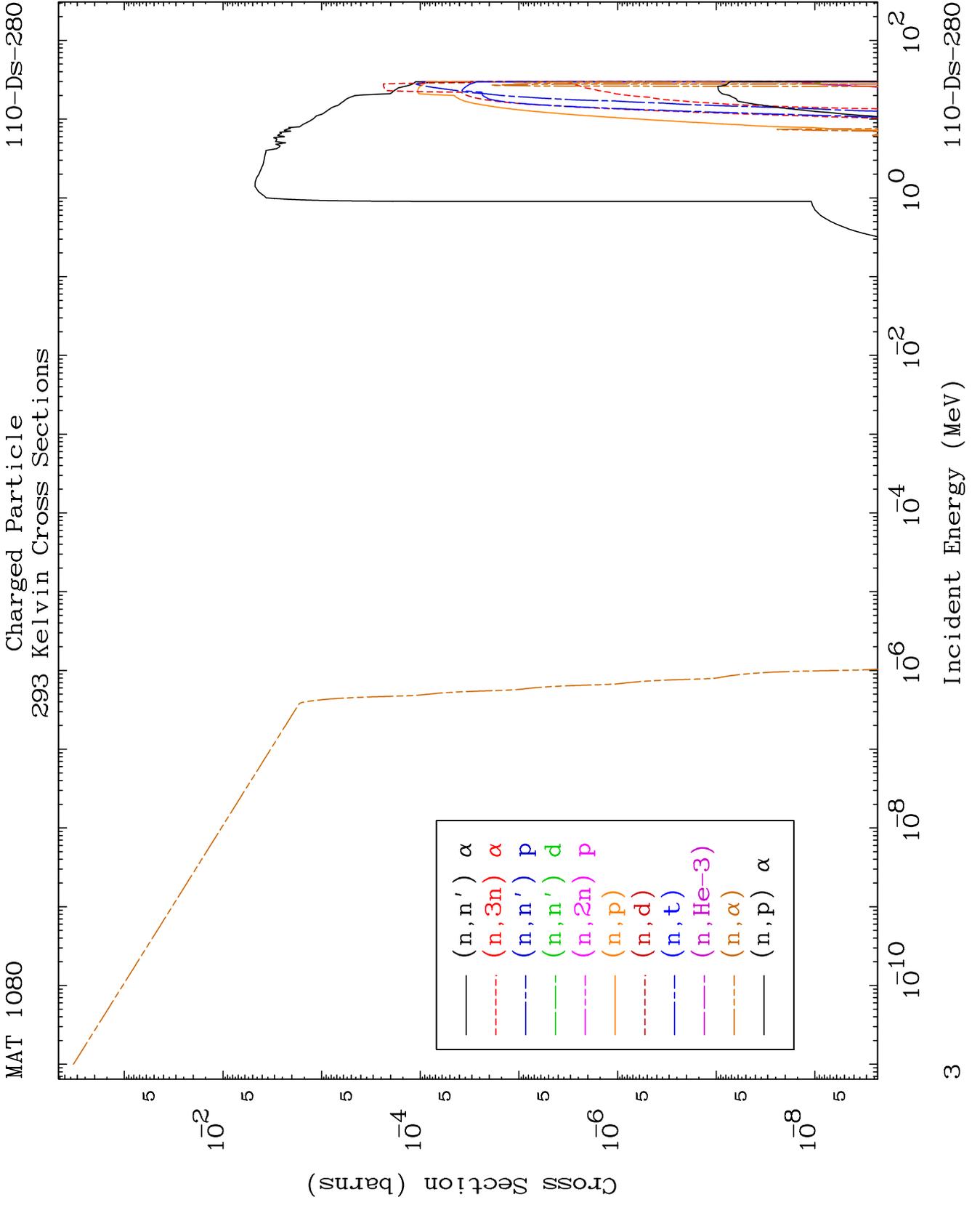
Press Mouse Button to Start

MAT 1080

Neutron Production
293 Kelvin Cross Sections

110-Ds-280

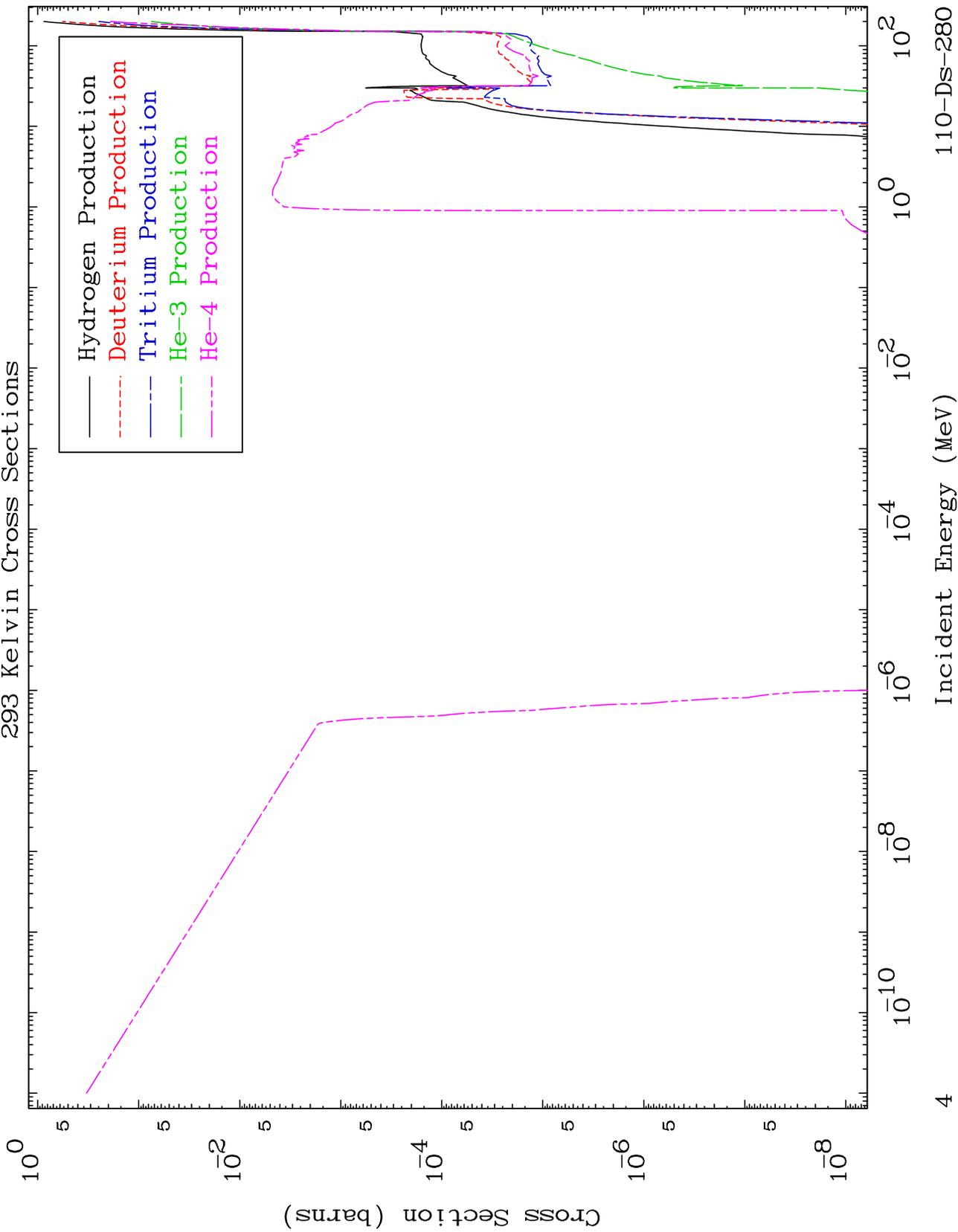




MAT 1080

Particle Production
293 Kelvin Cross Sections

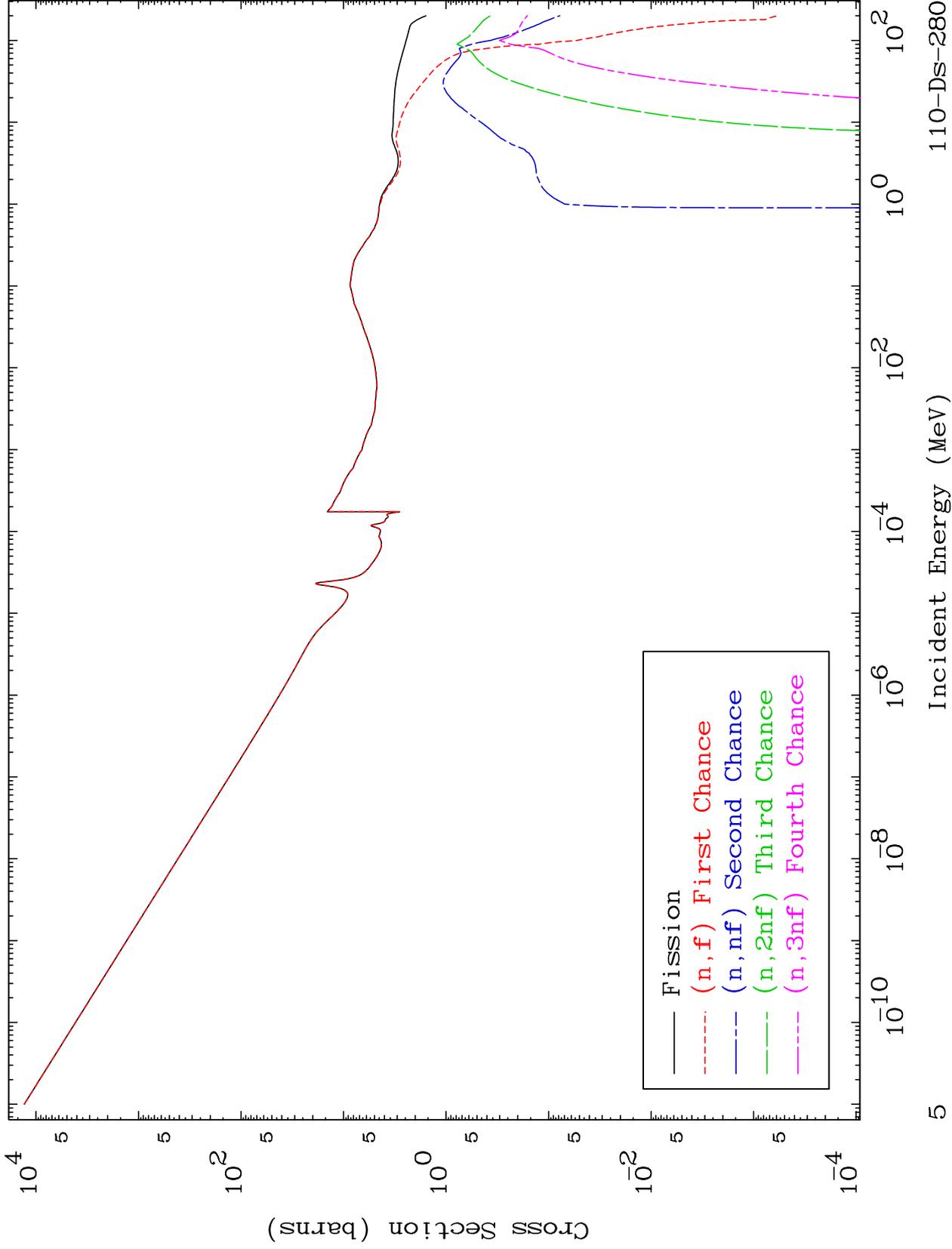
110-Ds-280



MAT 1080

Fission
293 Kelvin Cross Sections

110-Ds-280

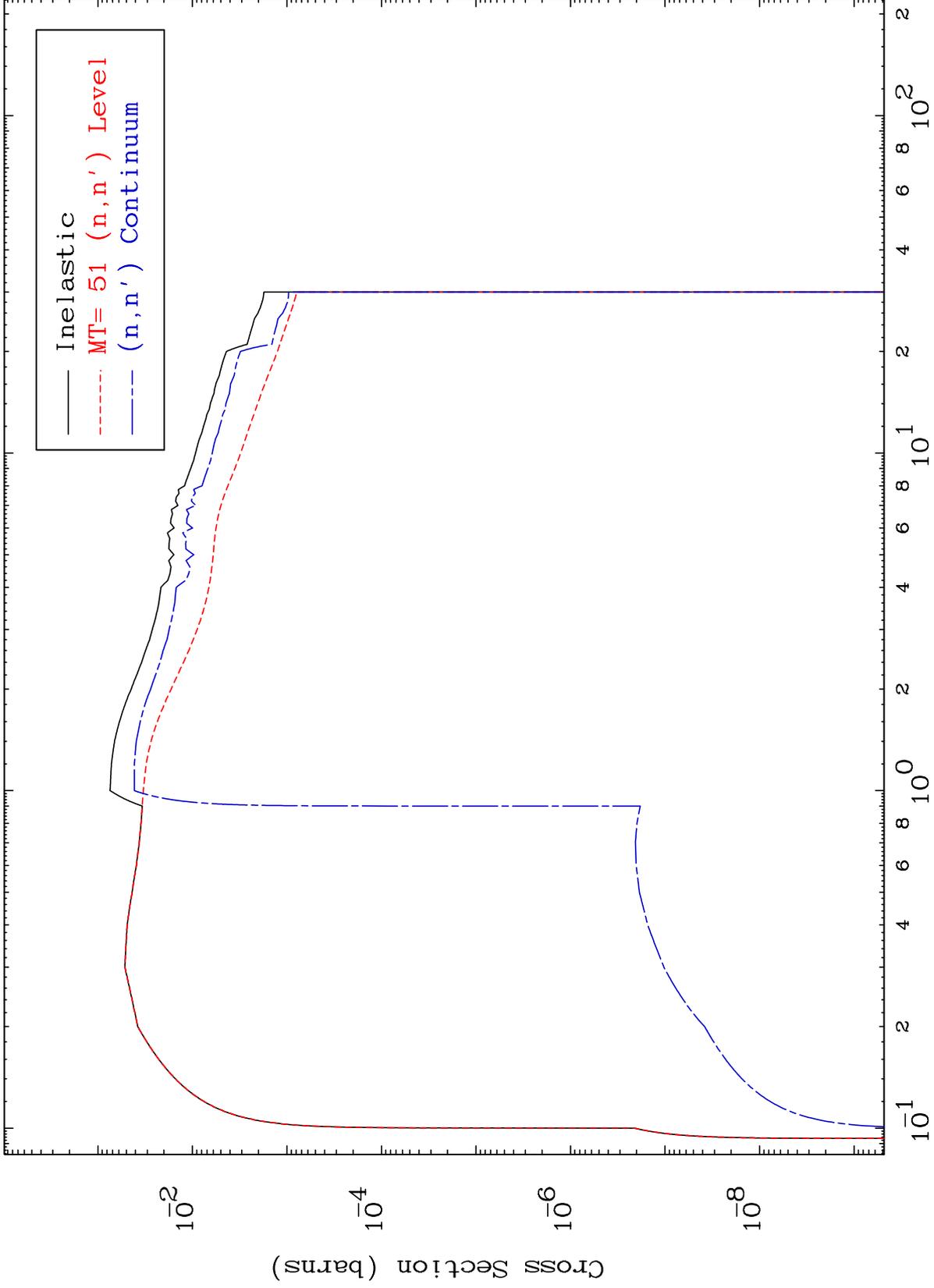


MAT 1080

(n,n') Level

110-Ds-280

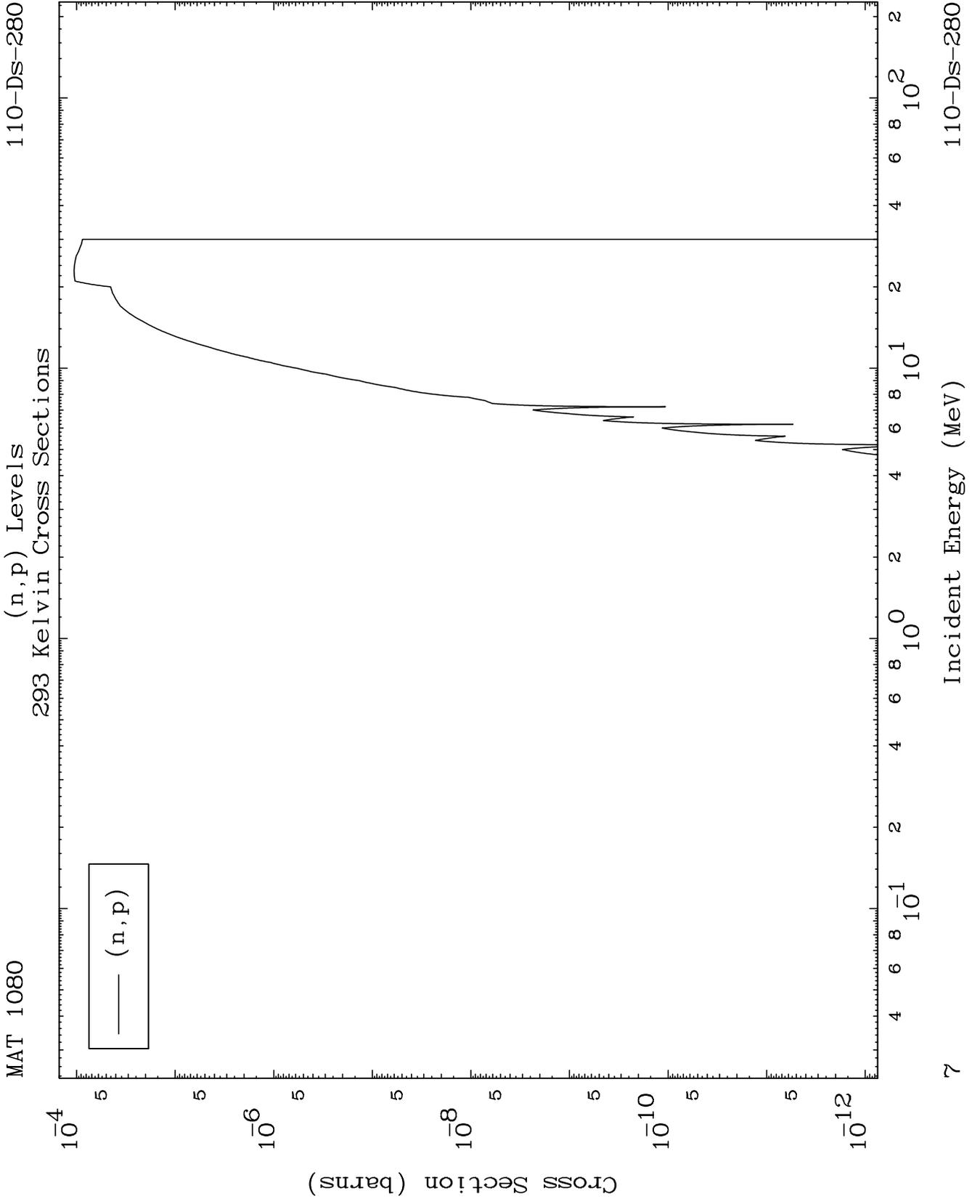
293 Kelvin Cross Sections



Incident Energy (MeV)

110-Ds-280

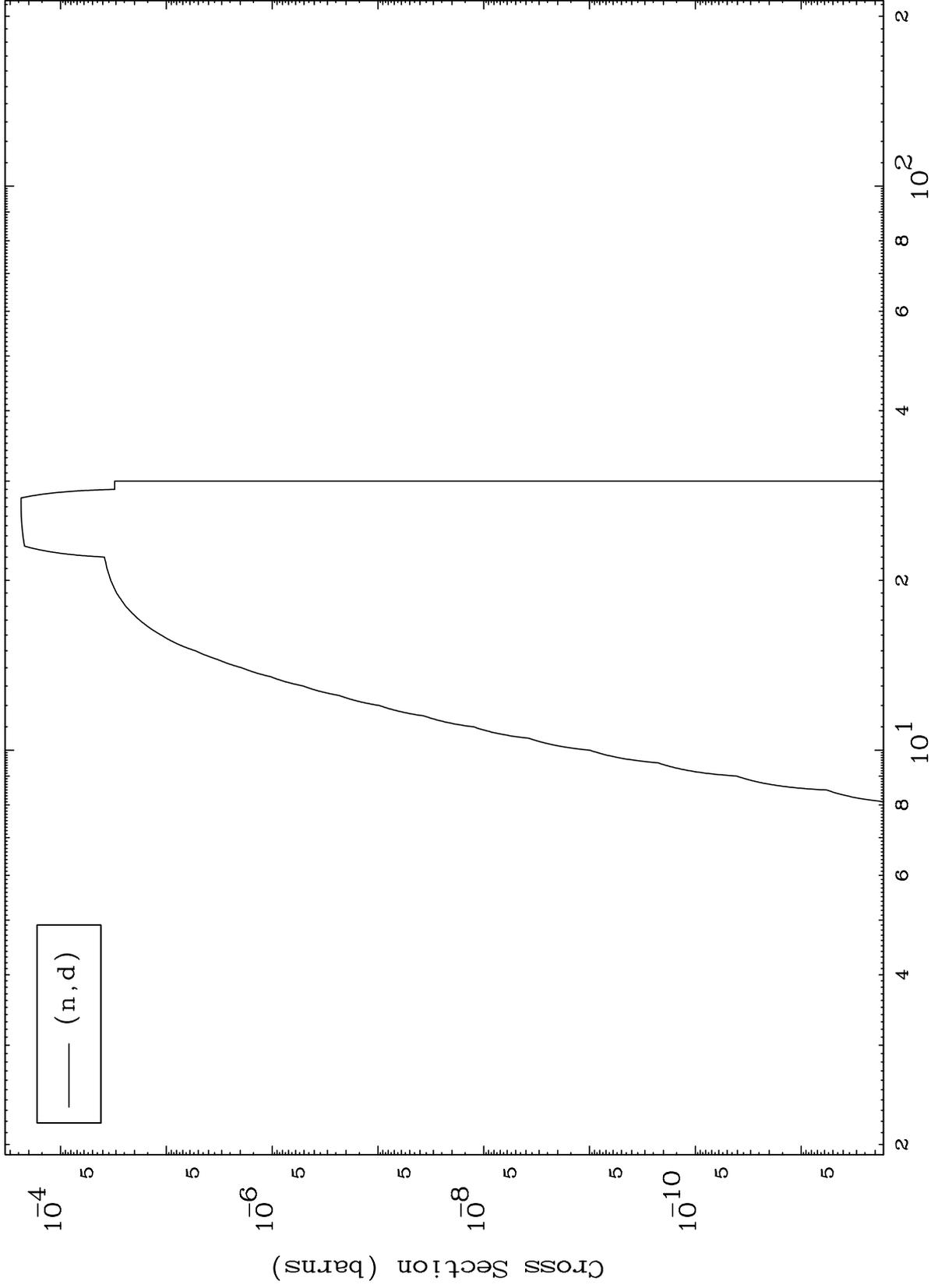
6



MAT 1080

(n,d) Levels
293 Kelvin Cross Sections

110-Ds-280



8

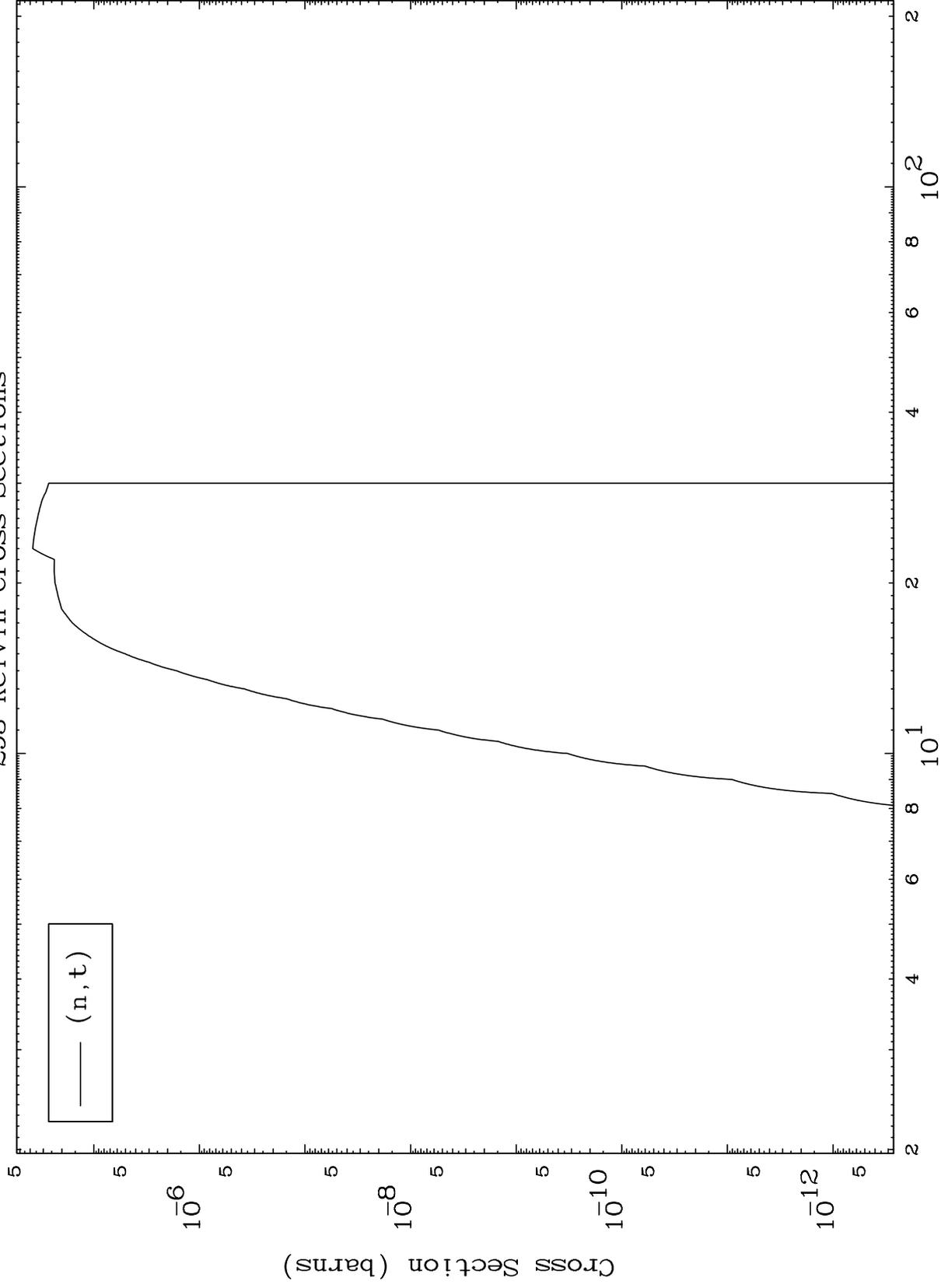
Incident Energy (MeV)

110-Ds-280

MAT 1080

(n,t) Levels
293 Kelvin Cross Sections

110-Ds-280



9

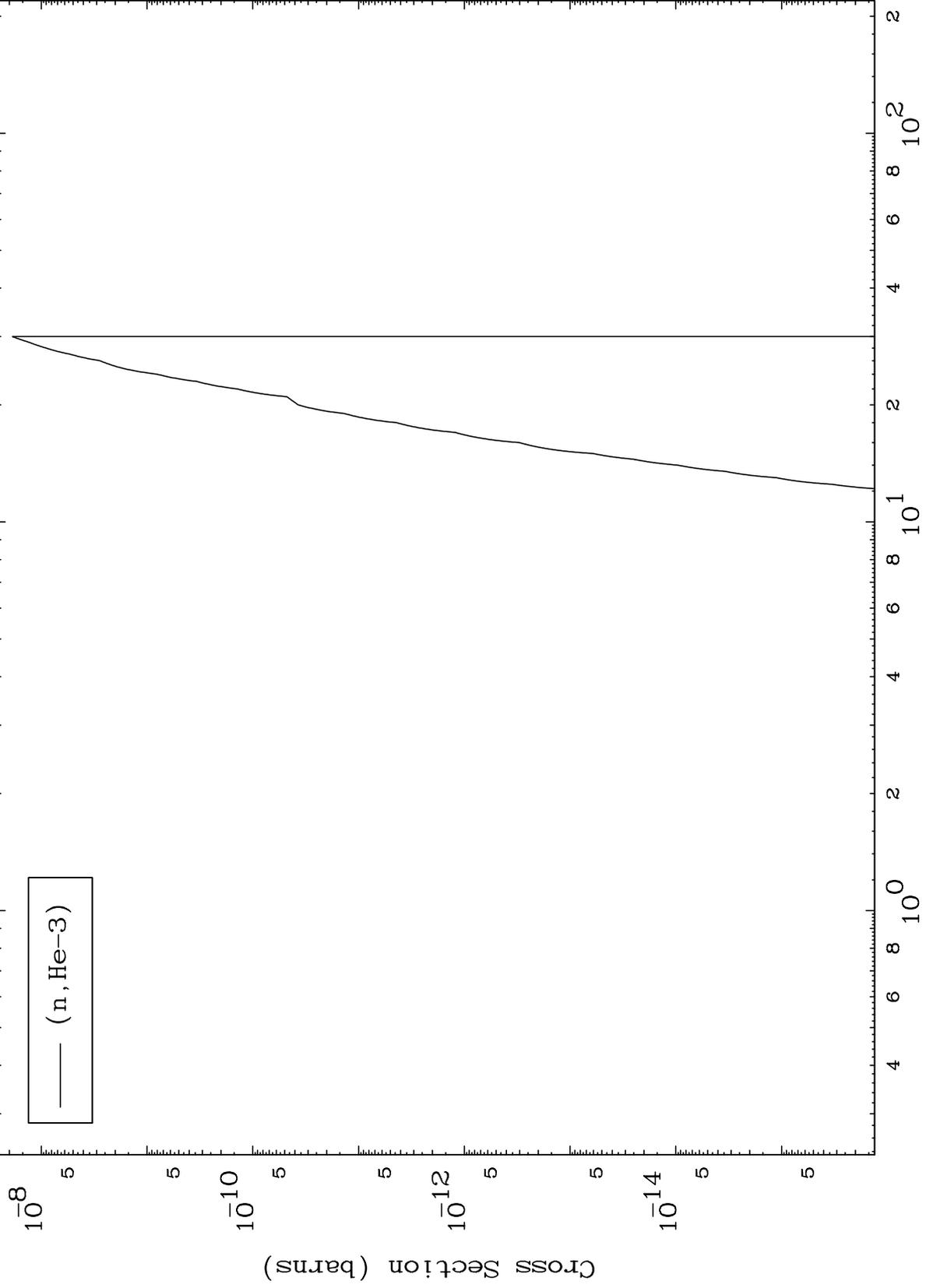
Incident Energy (MeV)

110-Ds-280

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(n,He3) Levels
293 Kelvin Cross Sections

110-Ds-280



10

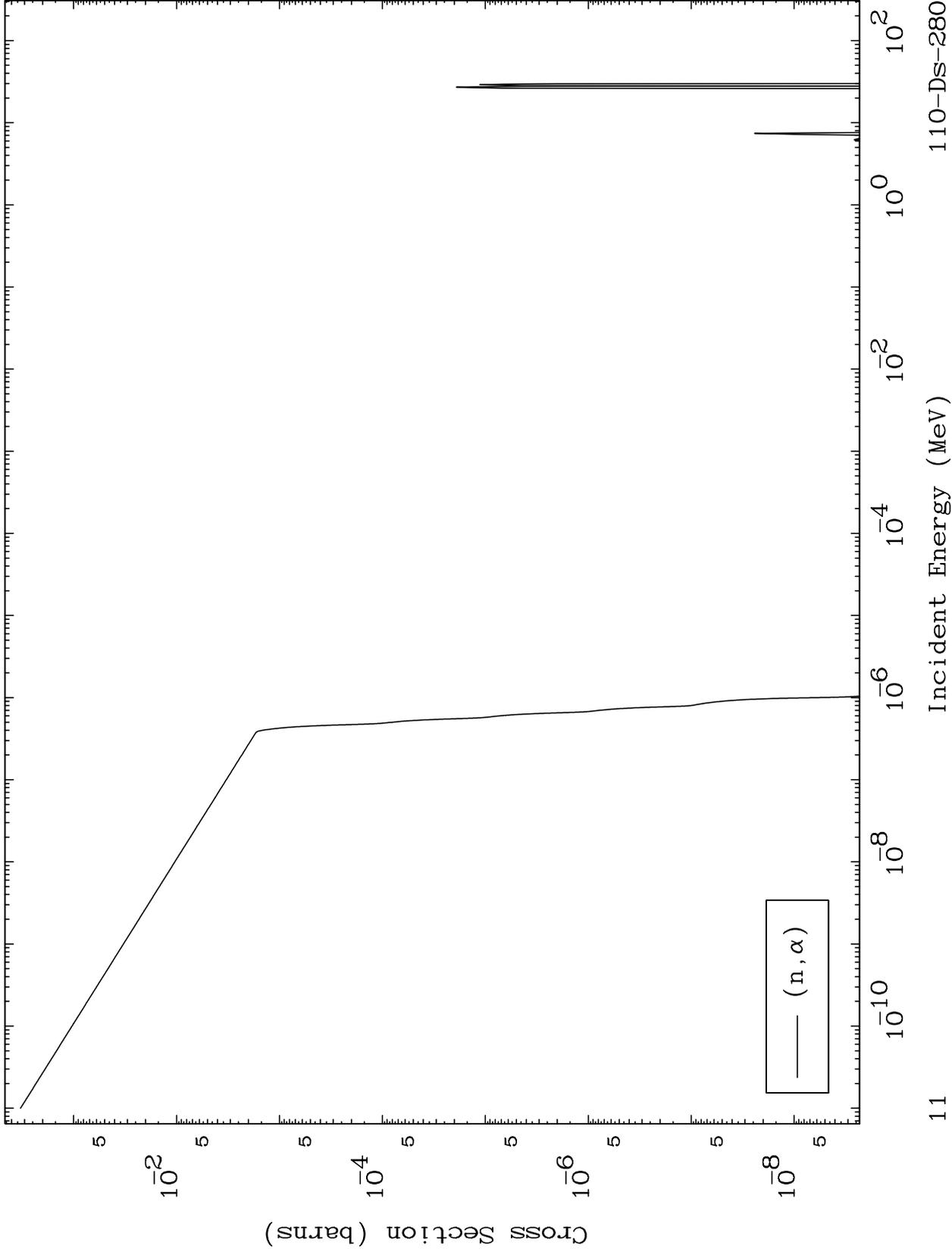
Incident Energy (MeV)

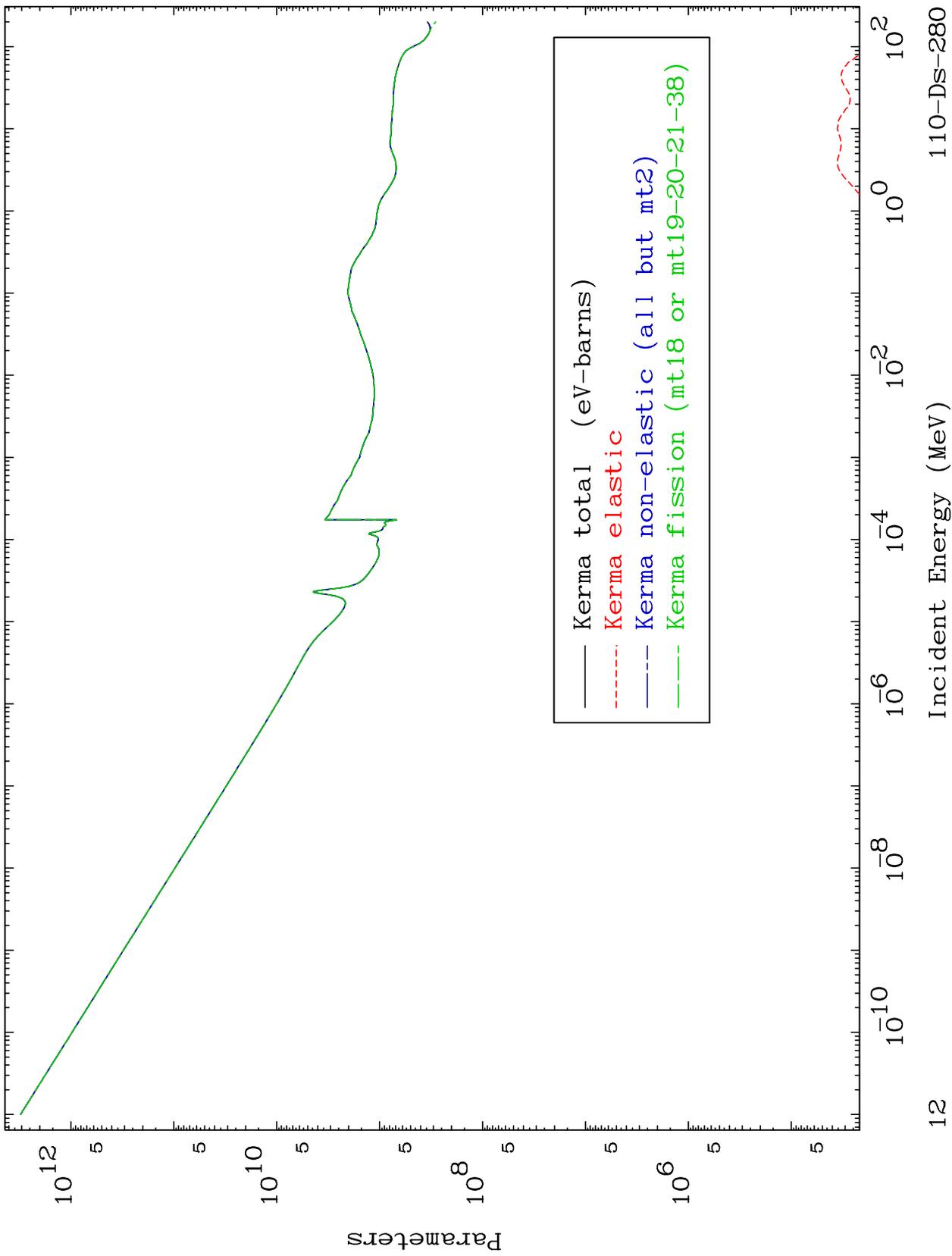
110-Ds-280

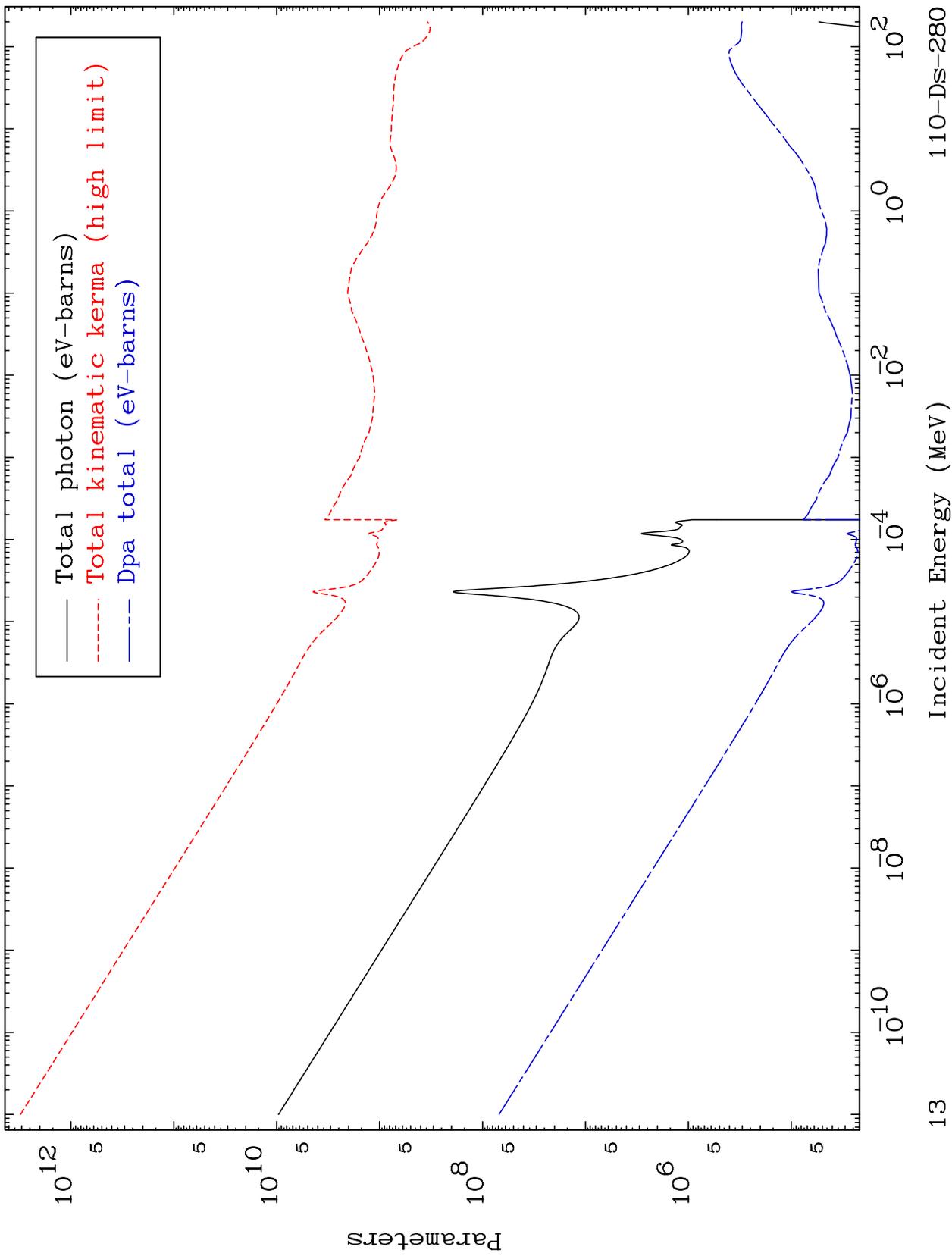
MAT 1080

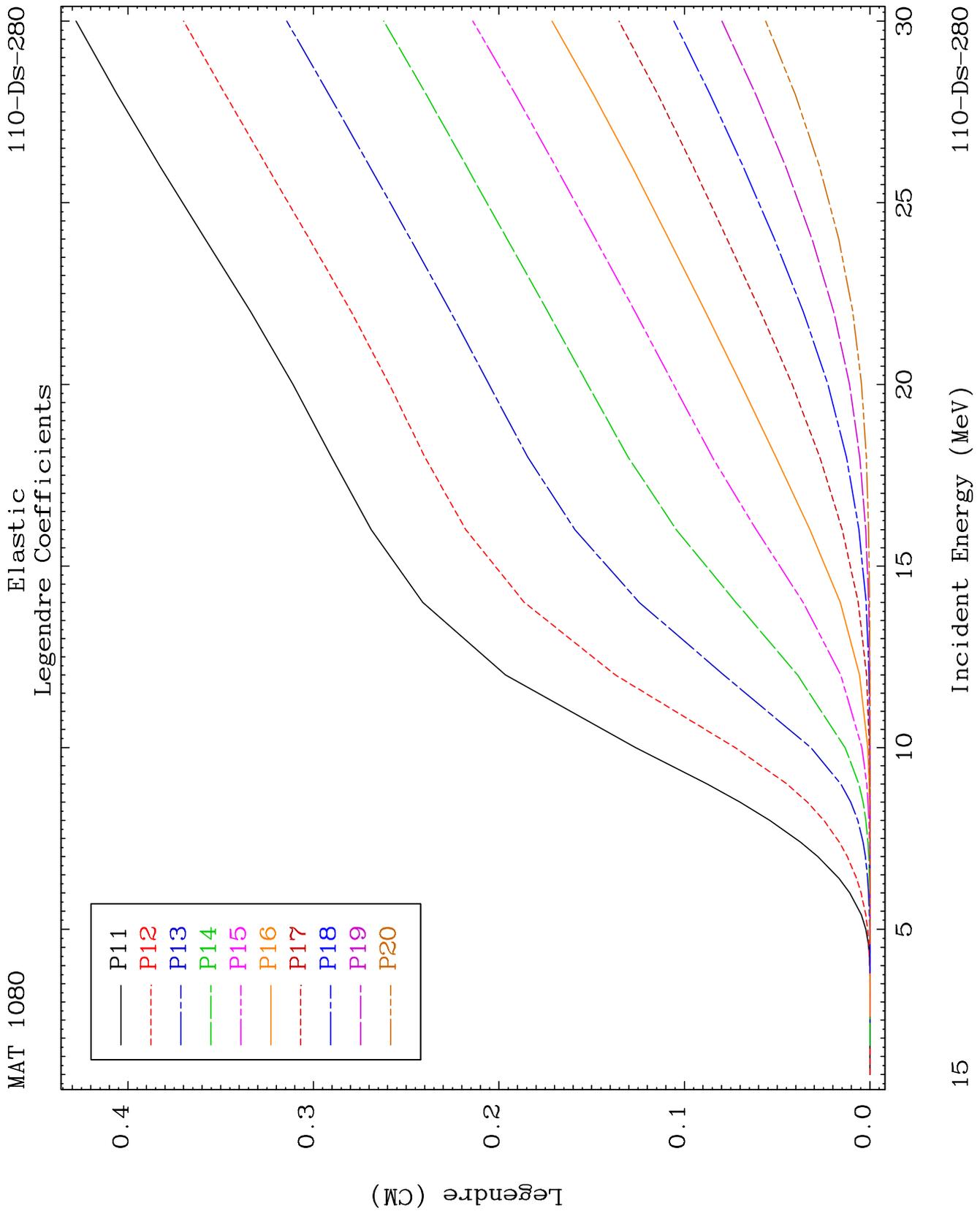
(n, α) Levels
293 Kelvin Cross Sections

110-Ds-280





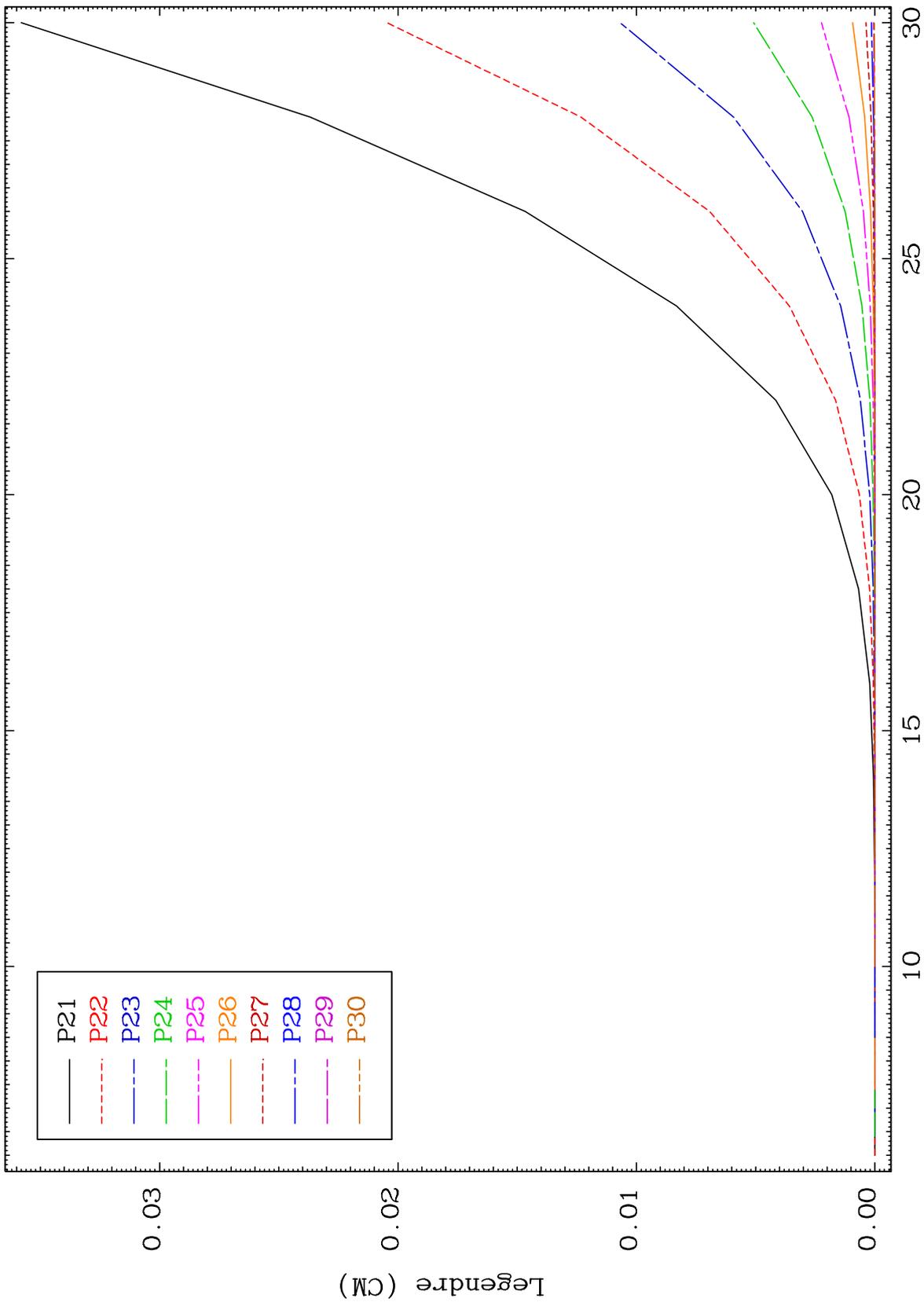




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Elastic Legendre Coefficients

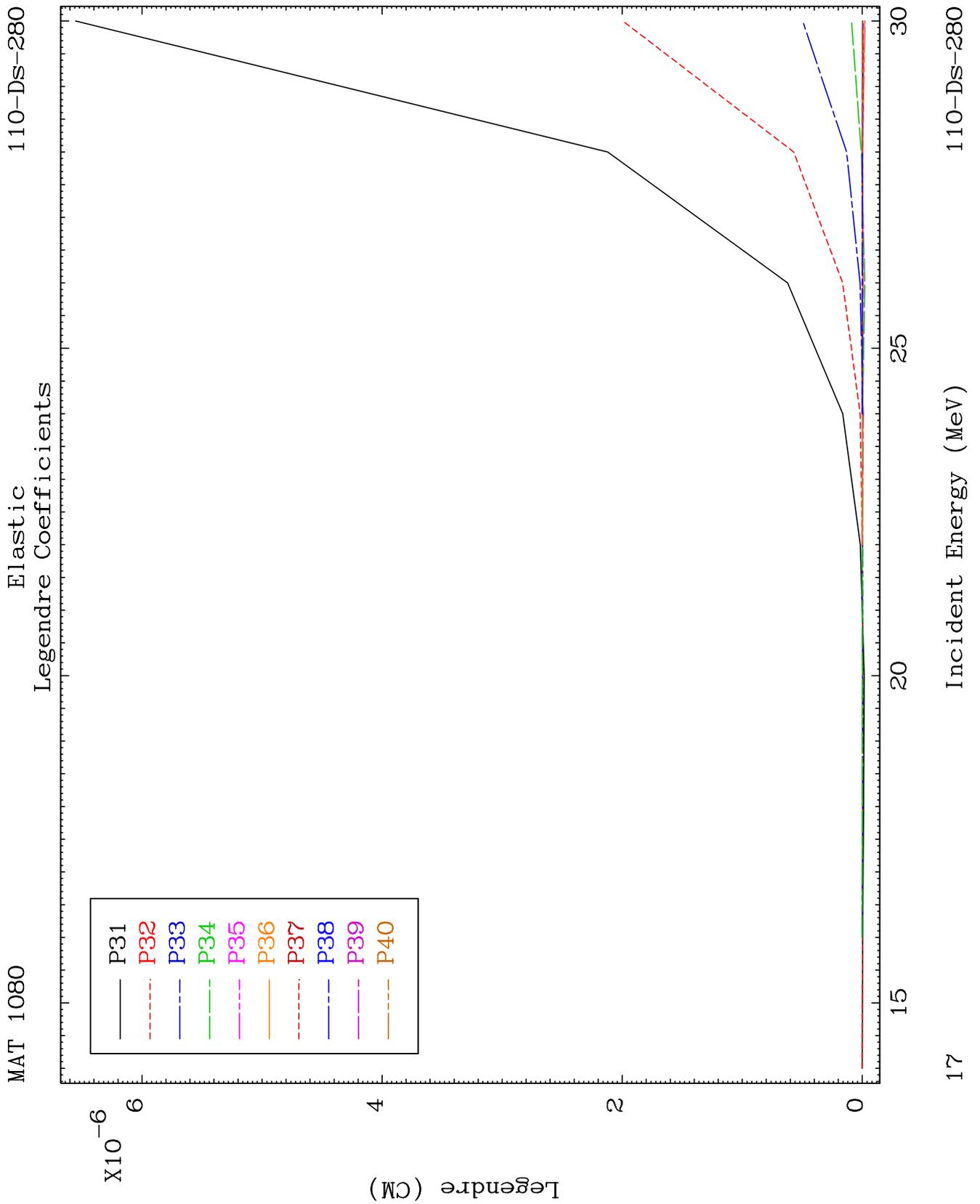
110-Ds-280



16

Incident Energy (MeV)

110-Ds-280



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Elastic Legendre Coefficients

110-Ds-280

17

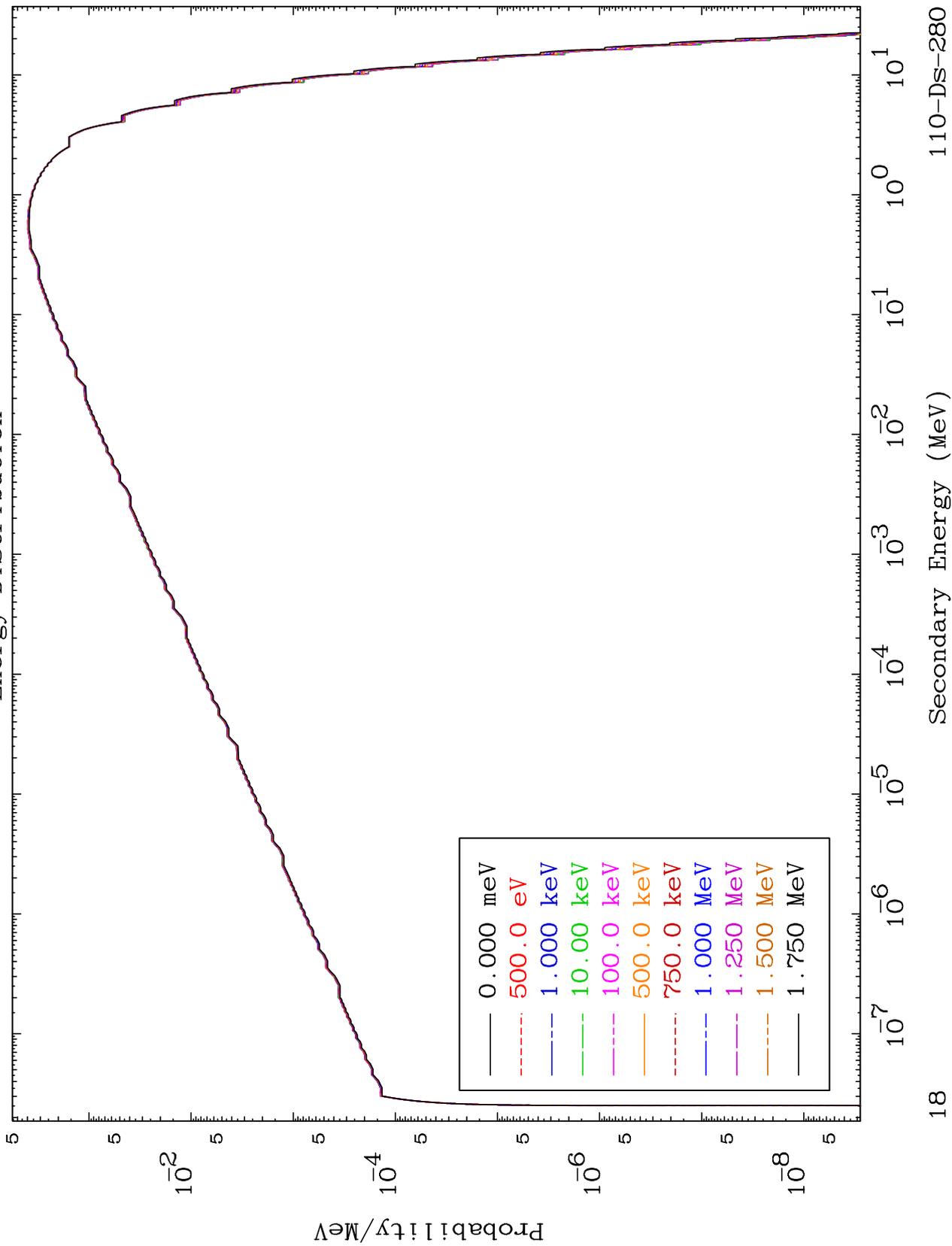
Incident Energy (MeV)

110-Ds-280

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Fission Energy Distribution

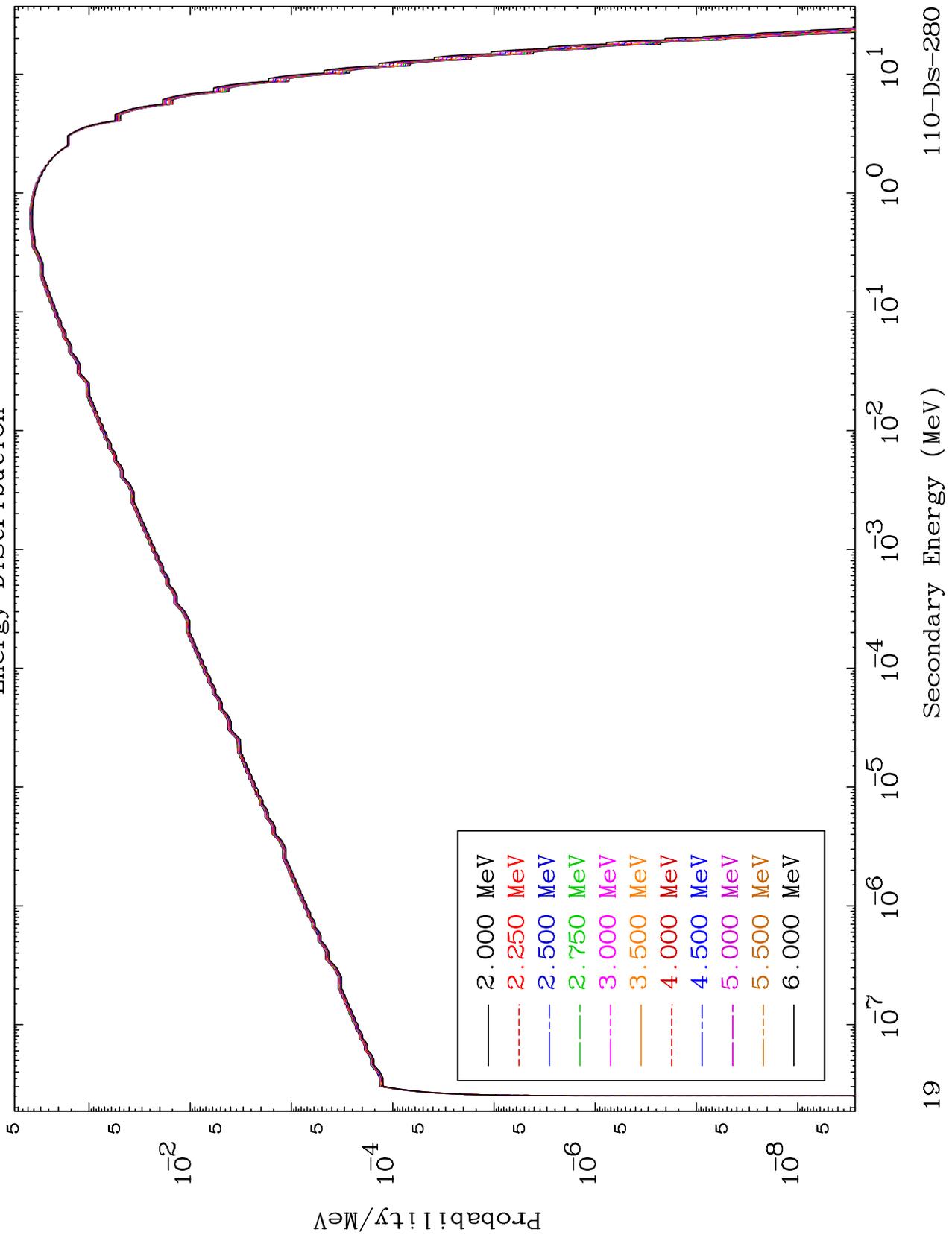
110-Ds-280



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Fission Energy Distribution

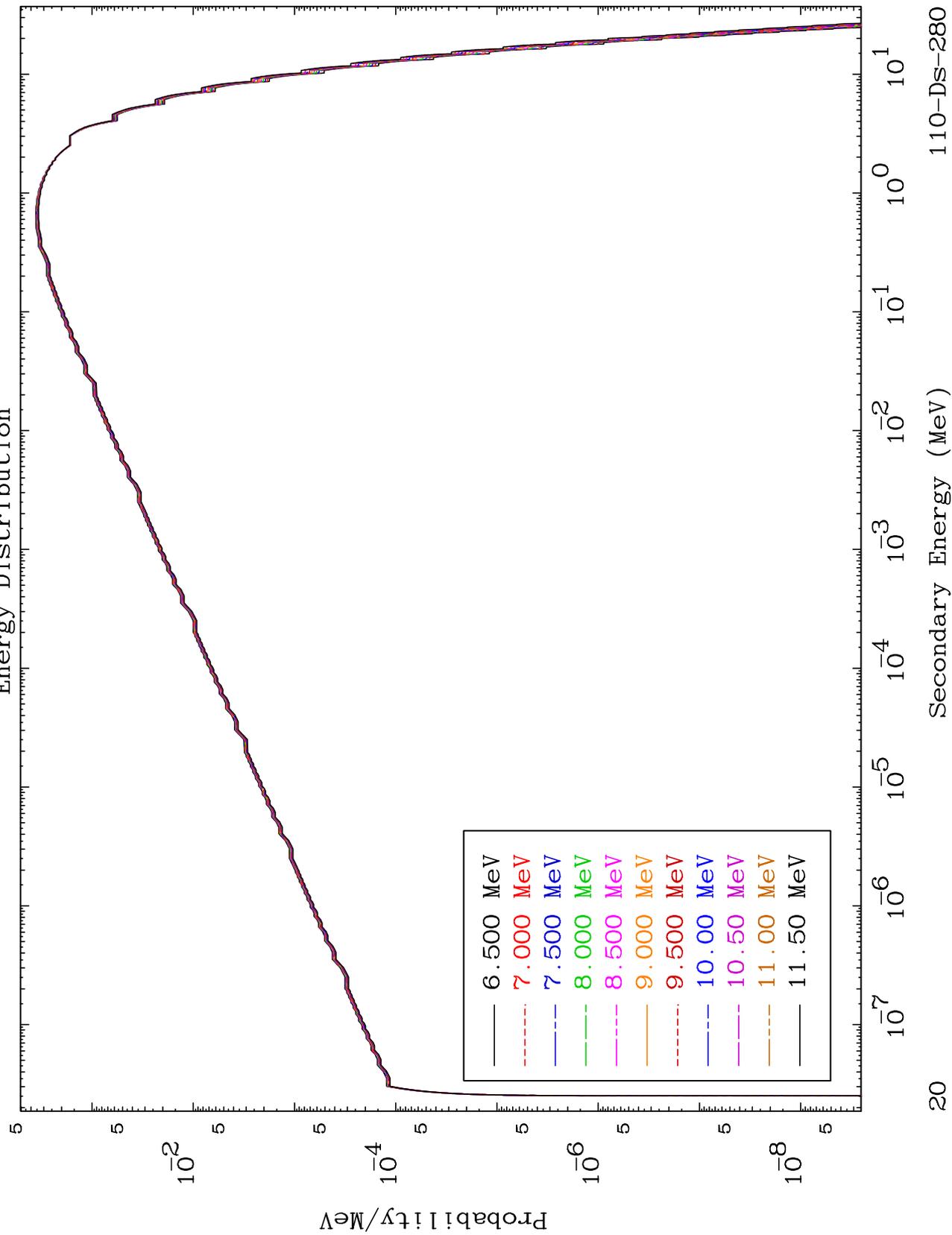
110-Ds-280



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Fission Energy Distribution

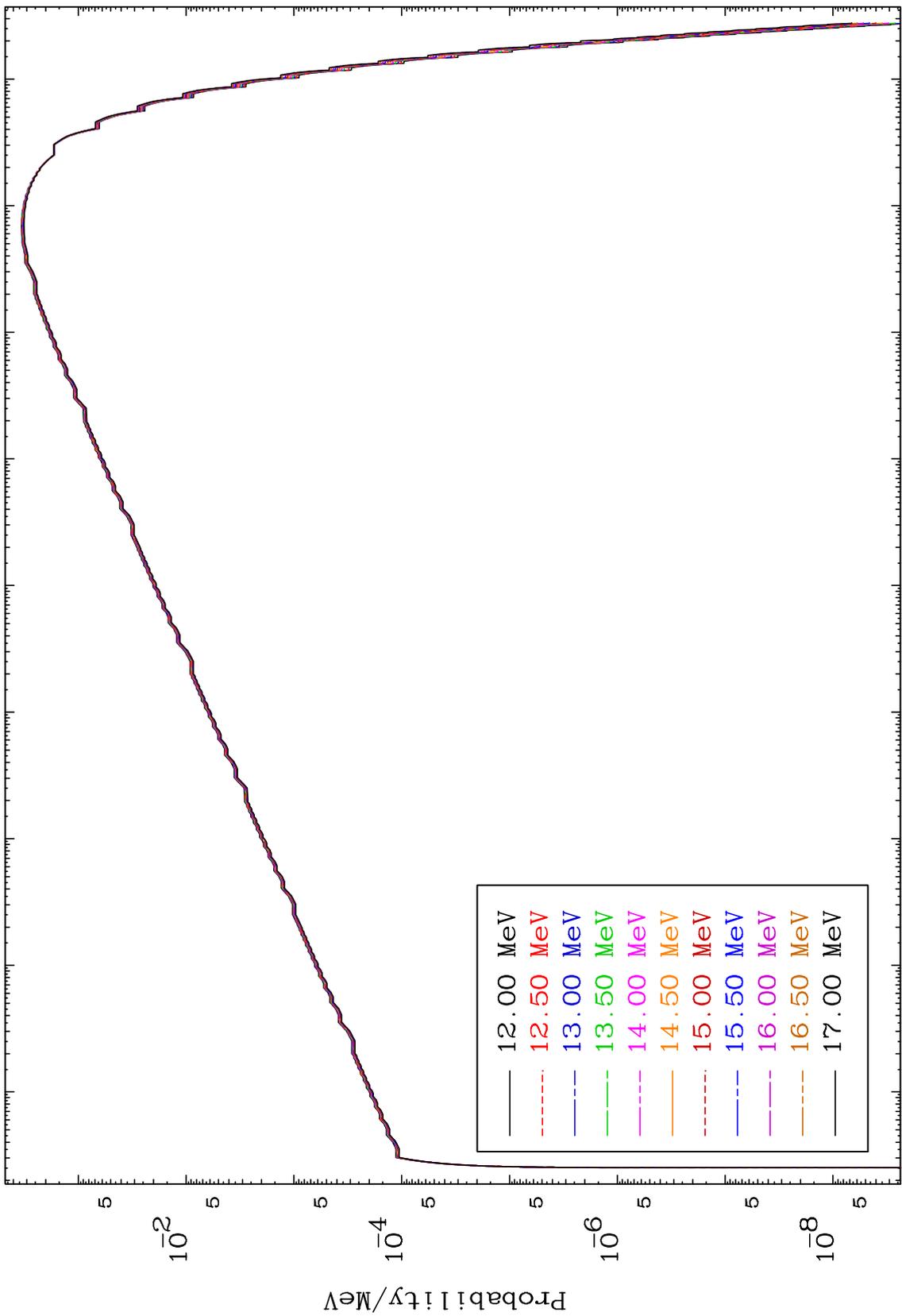
110-Ds-280



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Fission Energy Distribution

110-Ds-280



110-Ds-280

Secondary Energy (MeV)

Fission
Energy Distribution

