Program EVALPLOT
(Version 2017-1)
by
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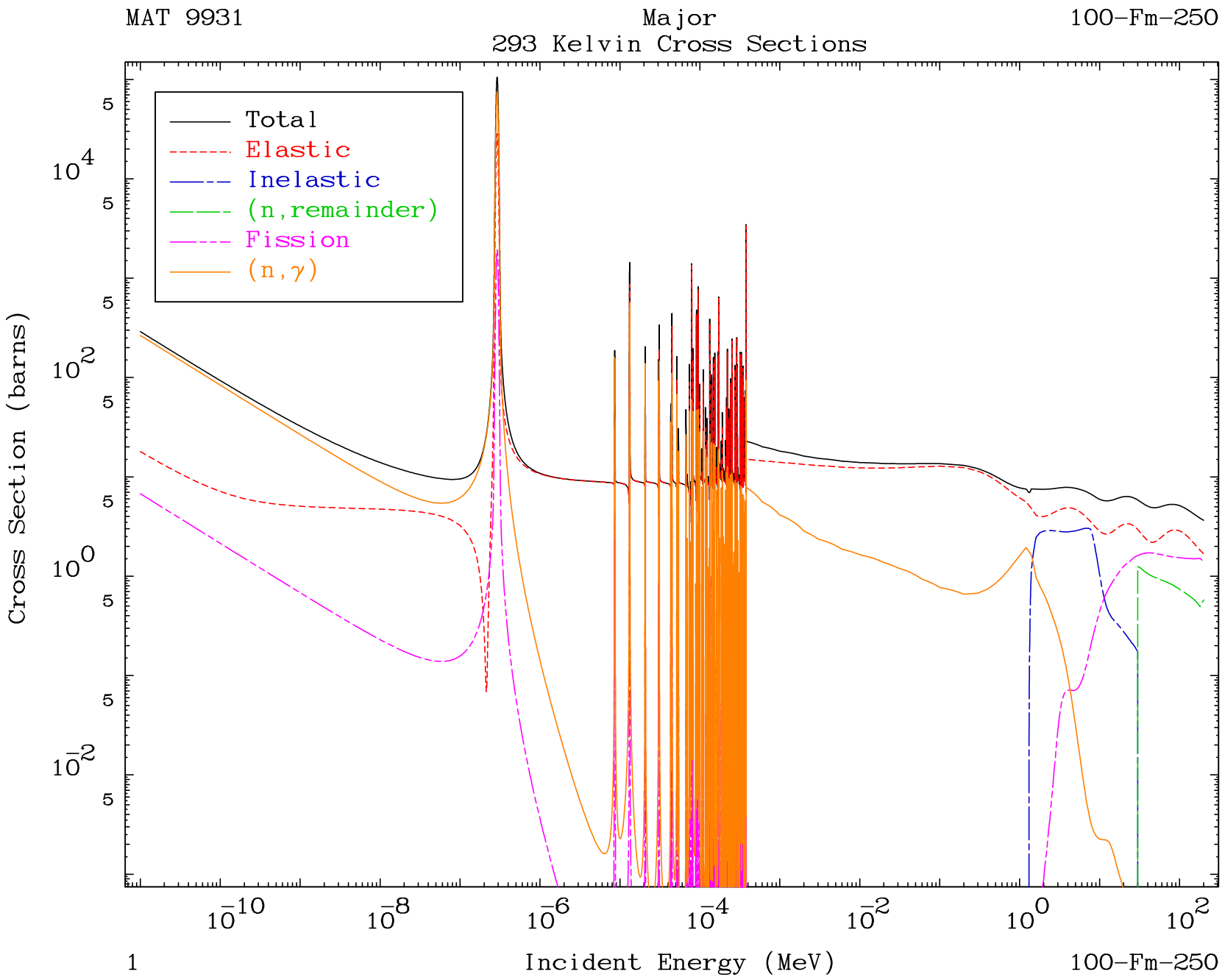
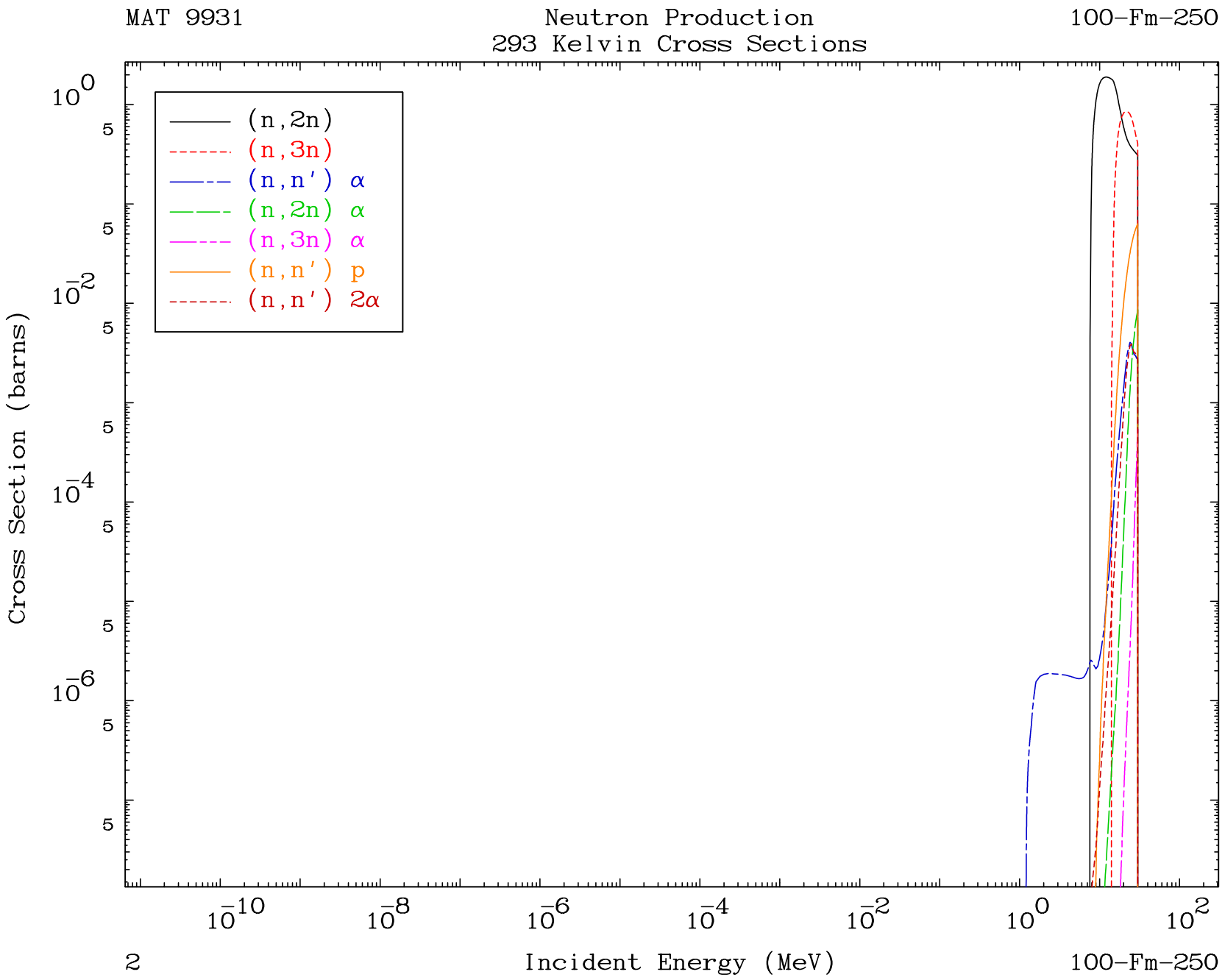
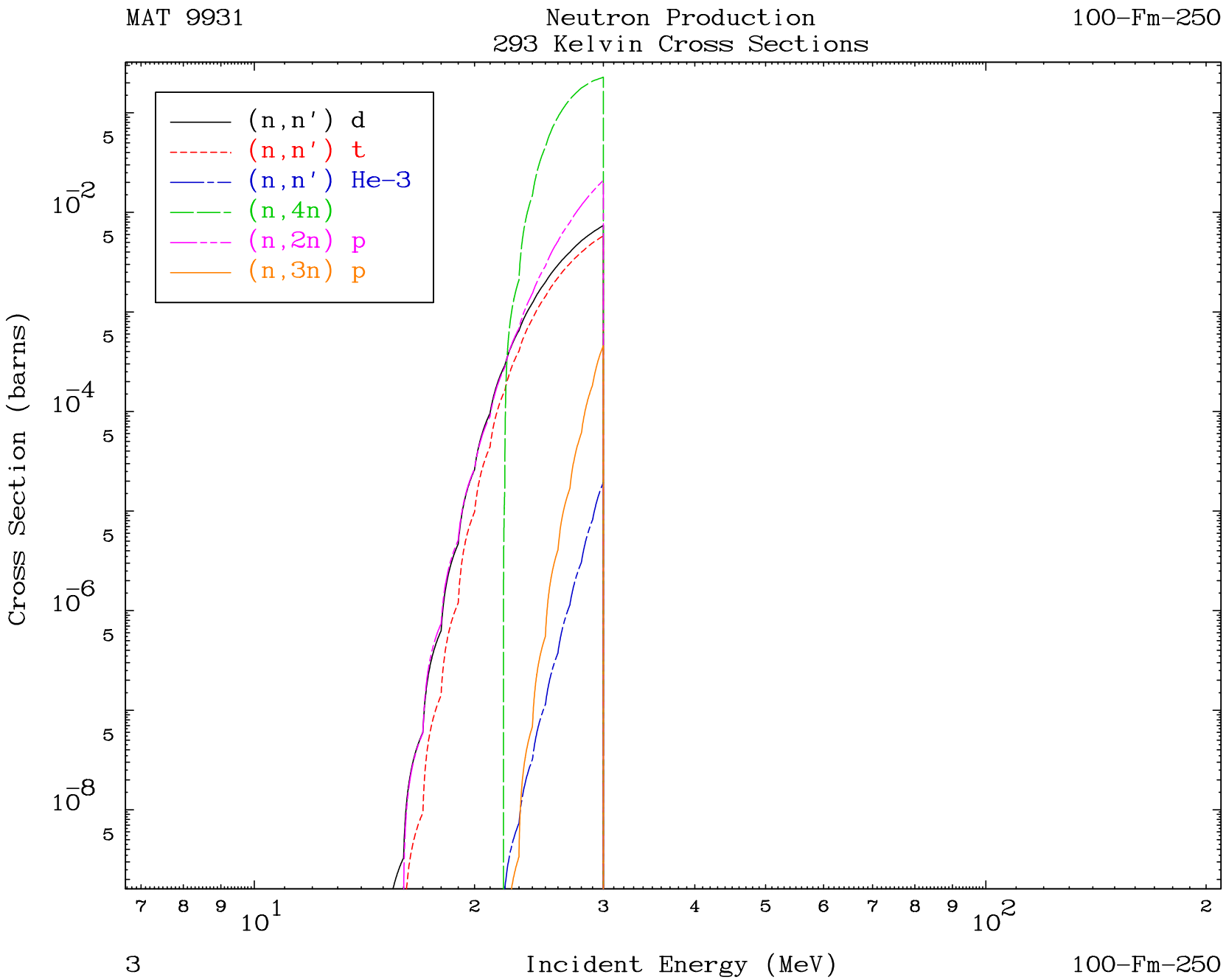
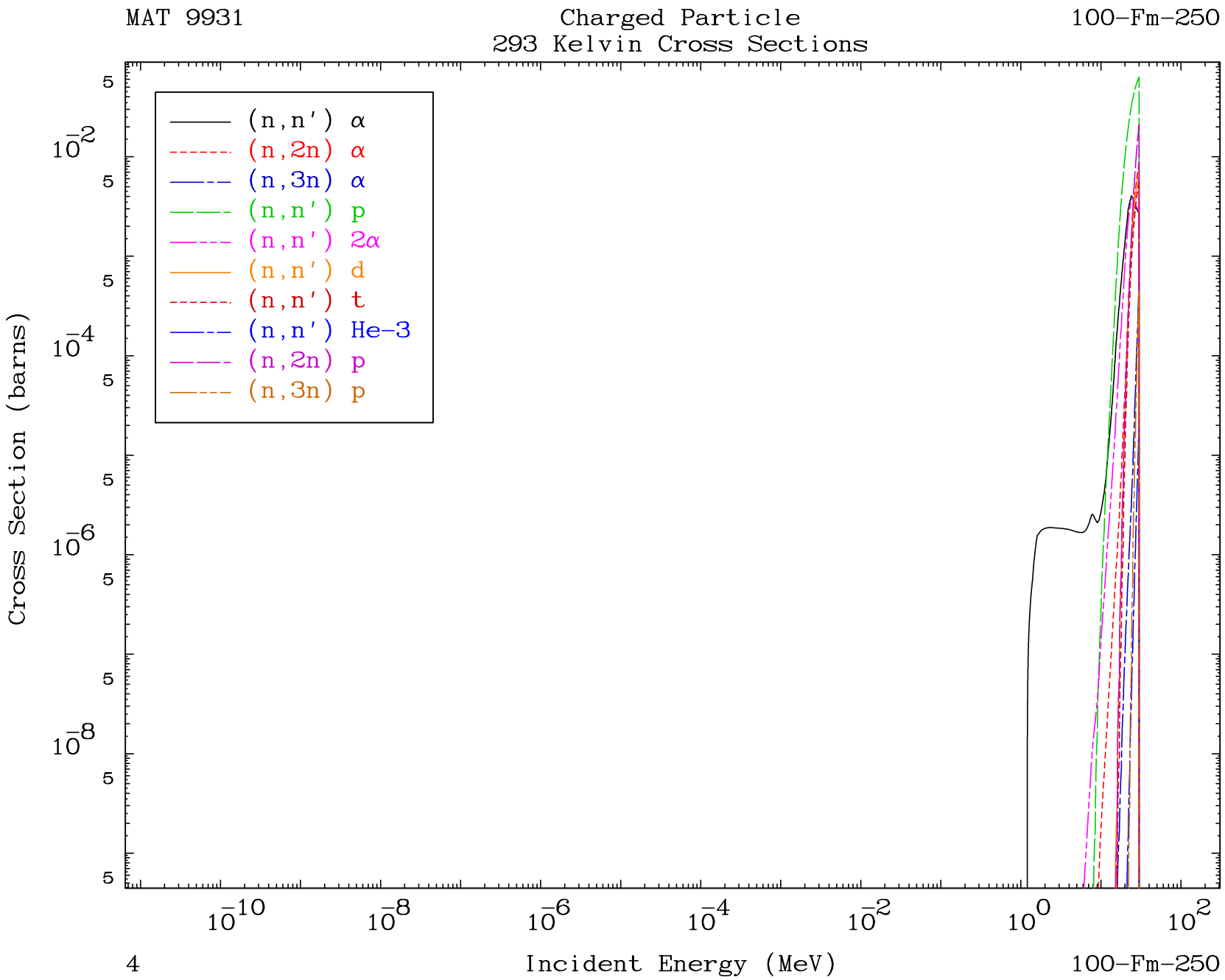
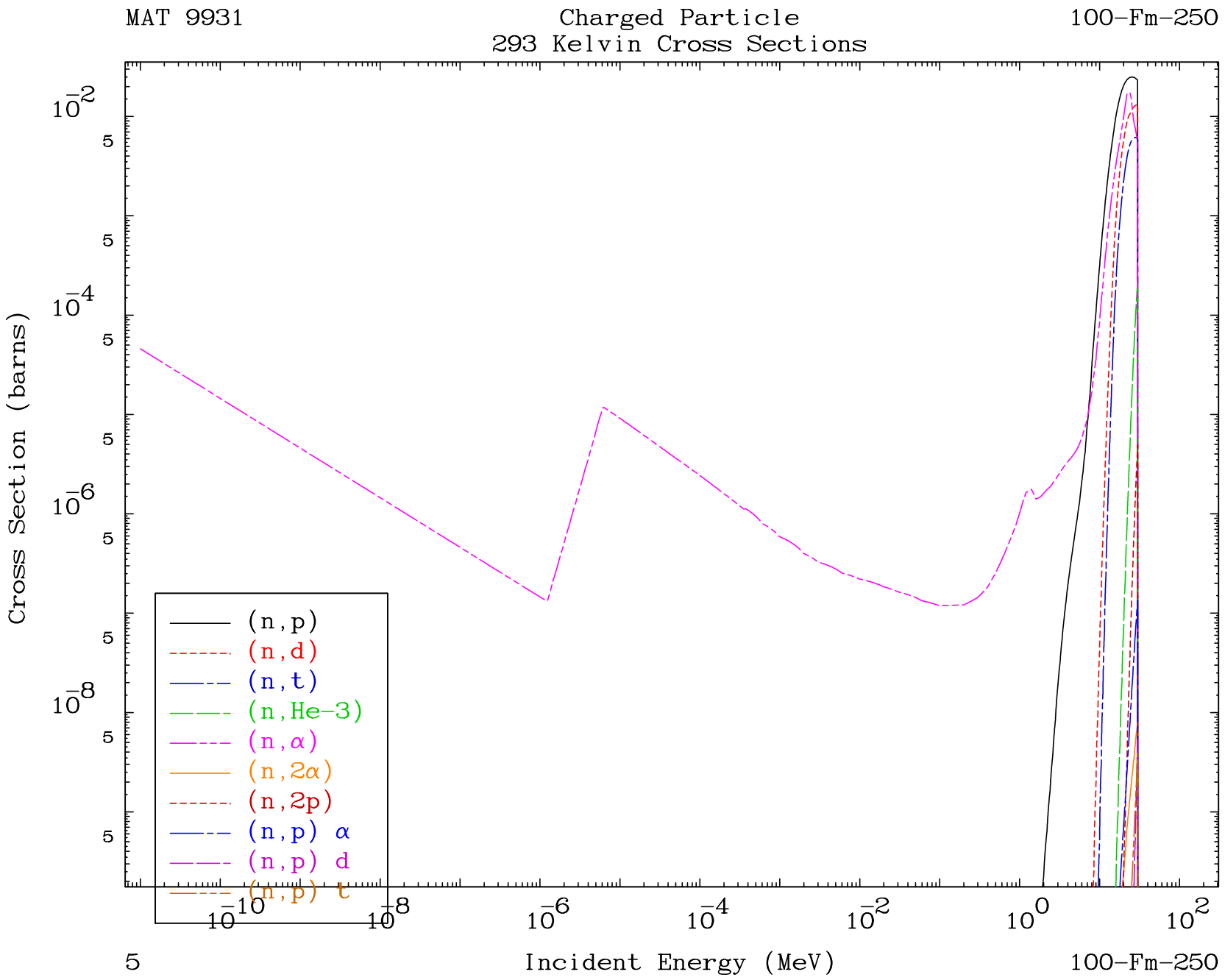
Press Mouse Button to Start

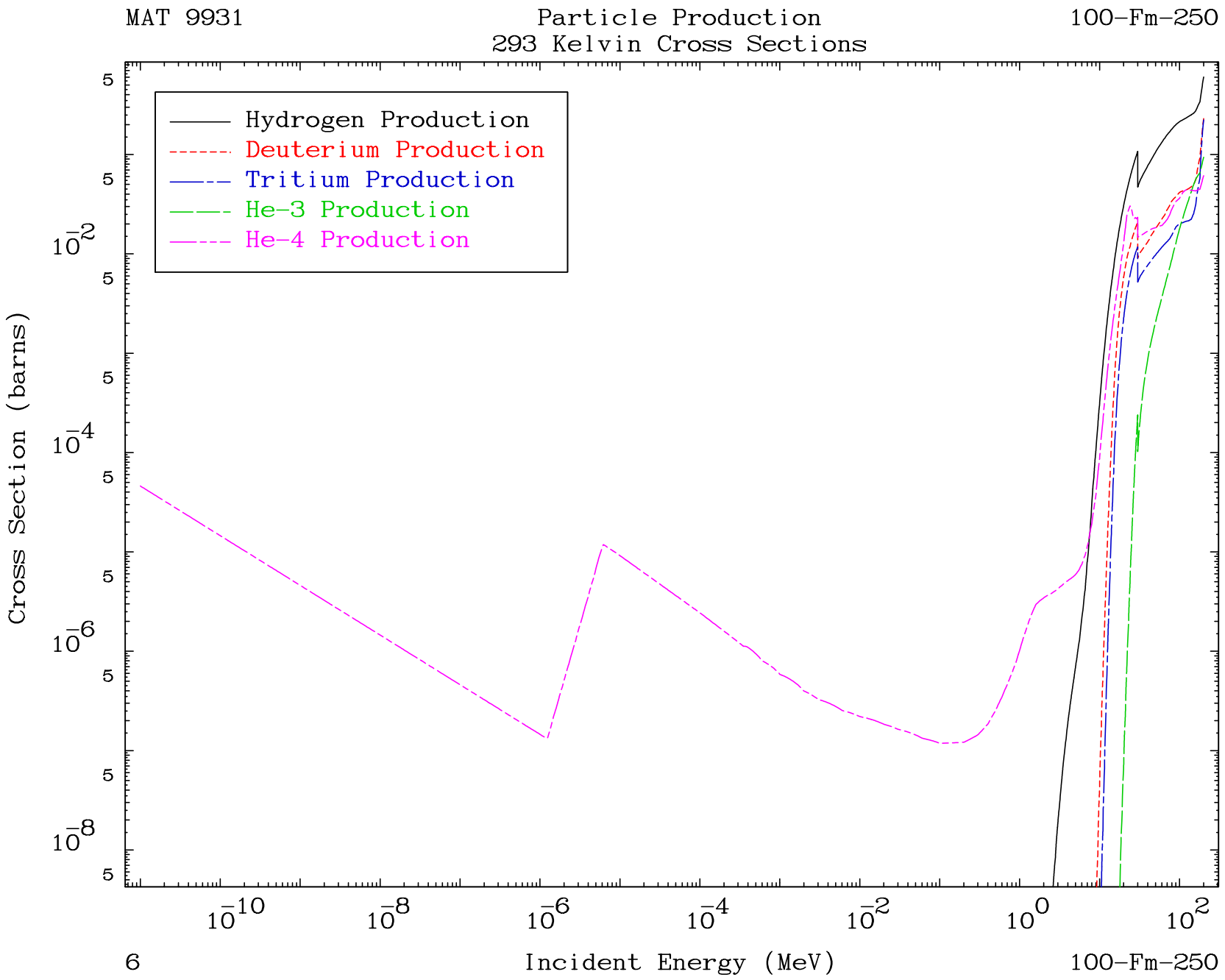

MAT 9931
Major
100-Fm-250
293 Kelvin Cross Sections
Total
Elastic
Inelastic
(n,remainder)
Fission
(n,γ)
Cross Section (barns)
Incident Energy (MeV)
1
100-Fm-250

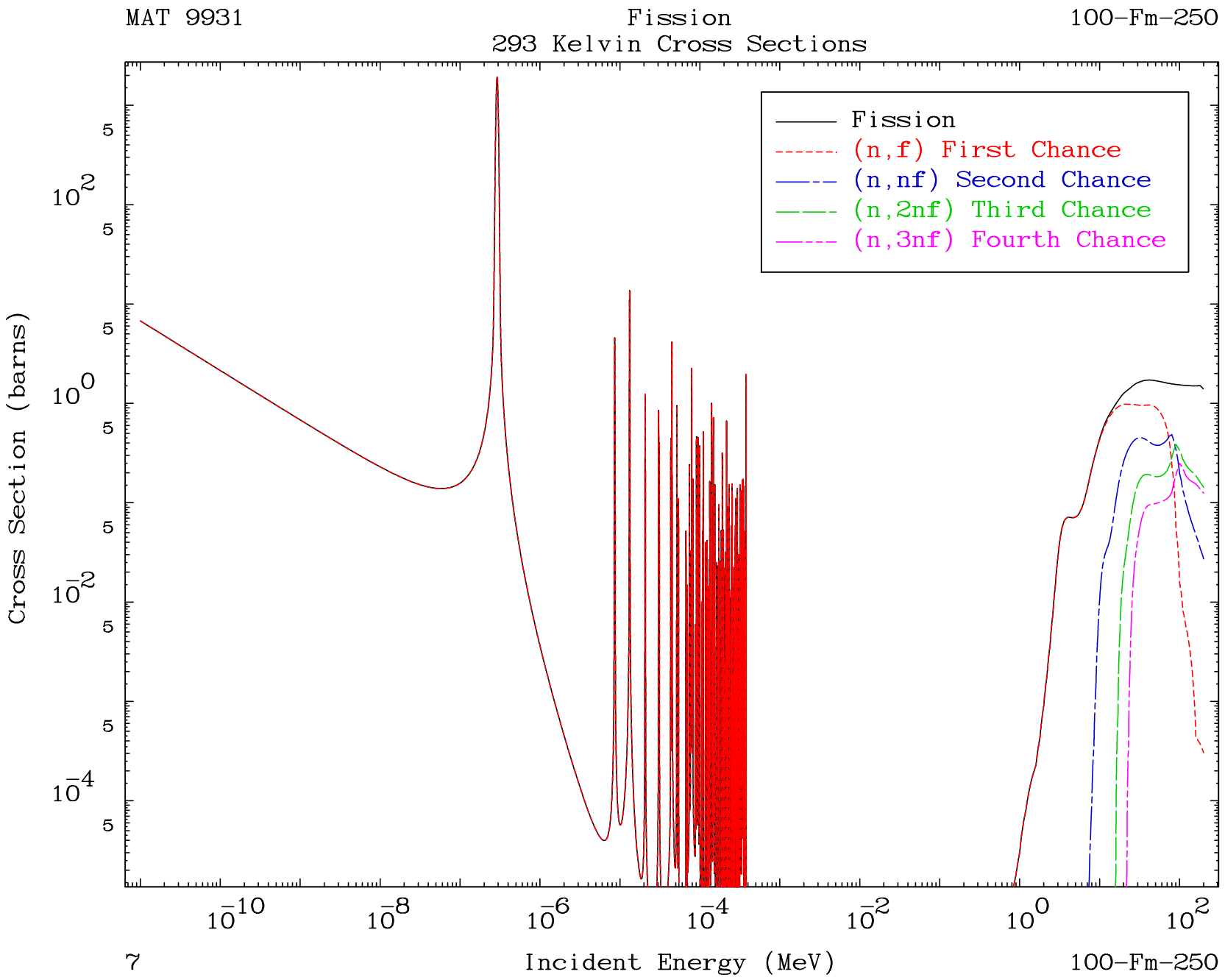

MAT 9931
Neutron Production
293 Kelvin Cross Sections
100-Fm-250
(n,2n)
(n,3n)
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
(n,n') 2α
Cross Section (barns)
Incident Energy (MeV)
100-Fm-250
2

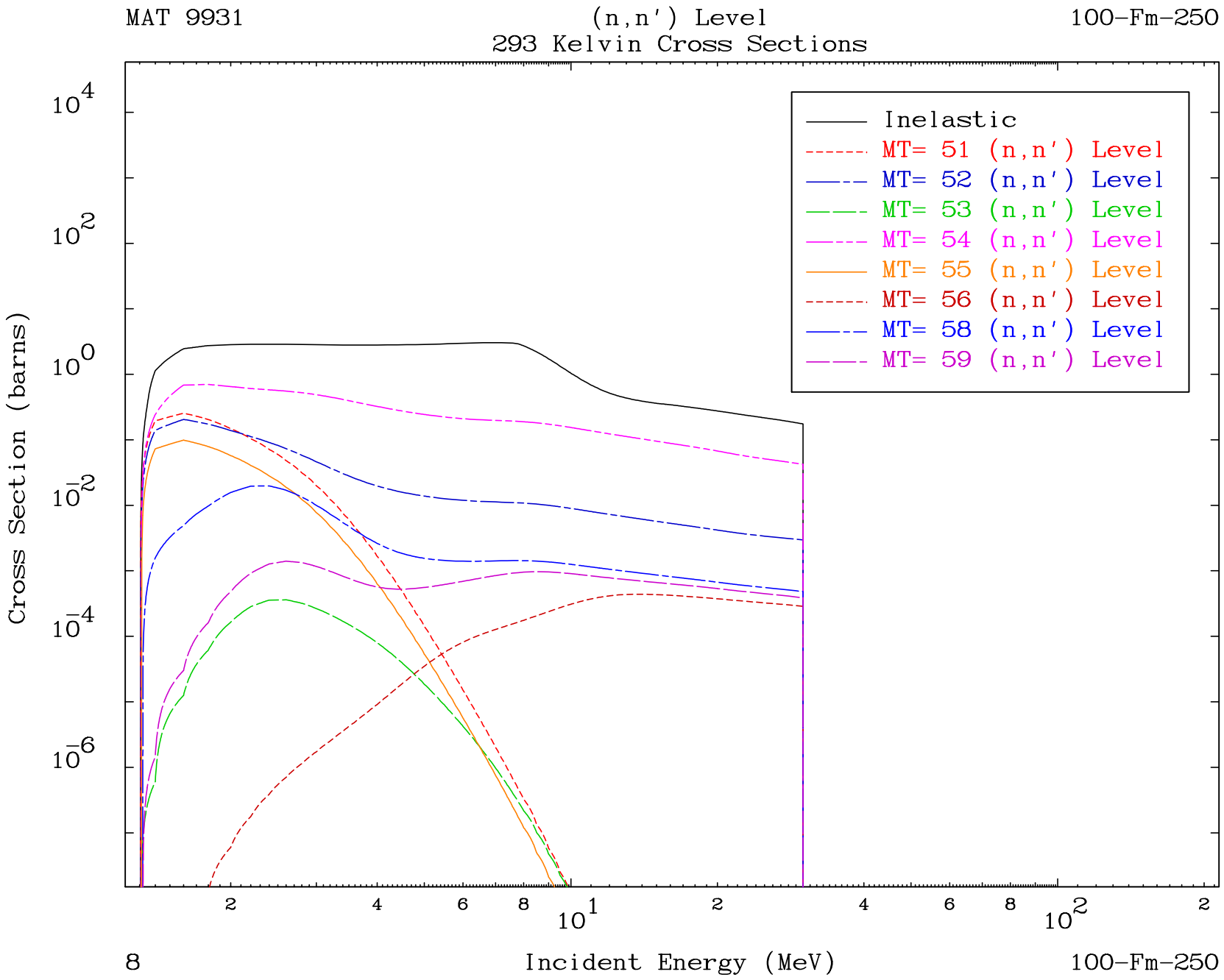

MAT 9931
Neutron Production
293 Kelvin Cross Sections
100-Fm-250
(n,n') d
(n,n') t
(n,n') He-3
(n,4n)
(n,2n) p
(n,3n) p
Cross Section (barns)
10-2
10-4
10-6
10-8
7 8 9 10 1 2 3 4 5 6 7 8 9 10 2 2
3
Incident Energy (MeV)
100-Fm-250

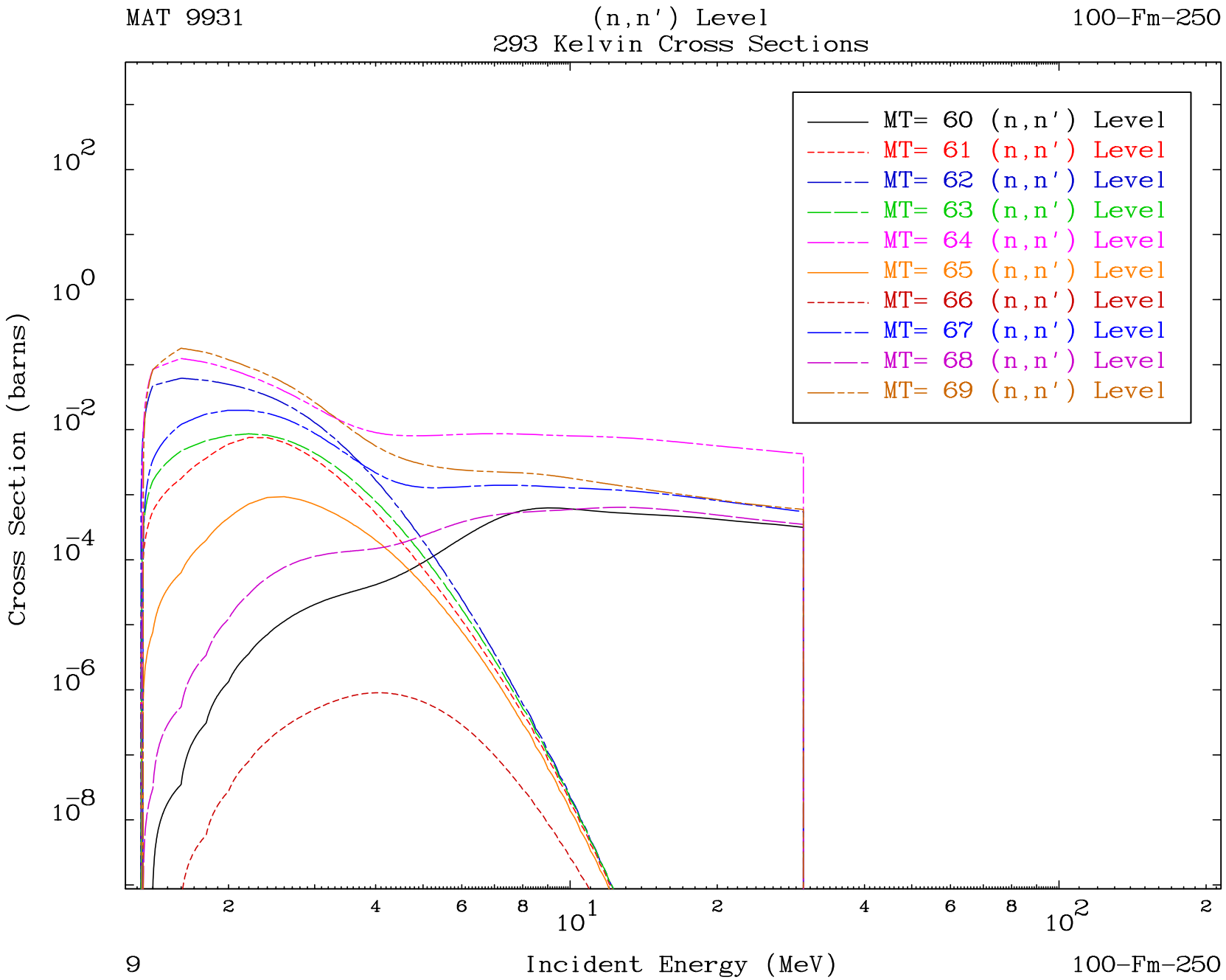

MAT 9931
Charged Particle
293 Kelvin Cross Sections
100-Fm-250
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
(n,n') 2α
(n,n') d
(n,n') t
(n,n') He-3
(n,2n) p
(n,3n) p
Cross Section (barns)
Incident Energy (MeV)
4
100-Fm-250

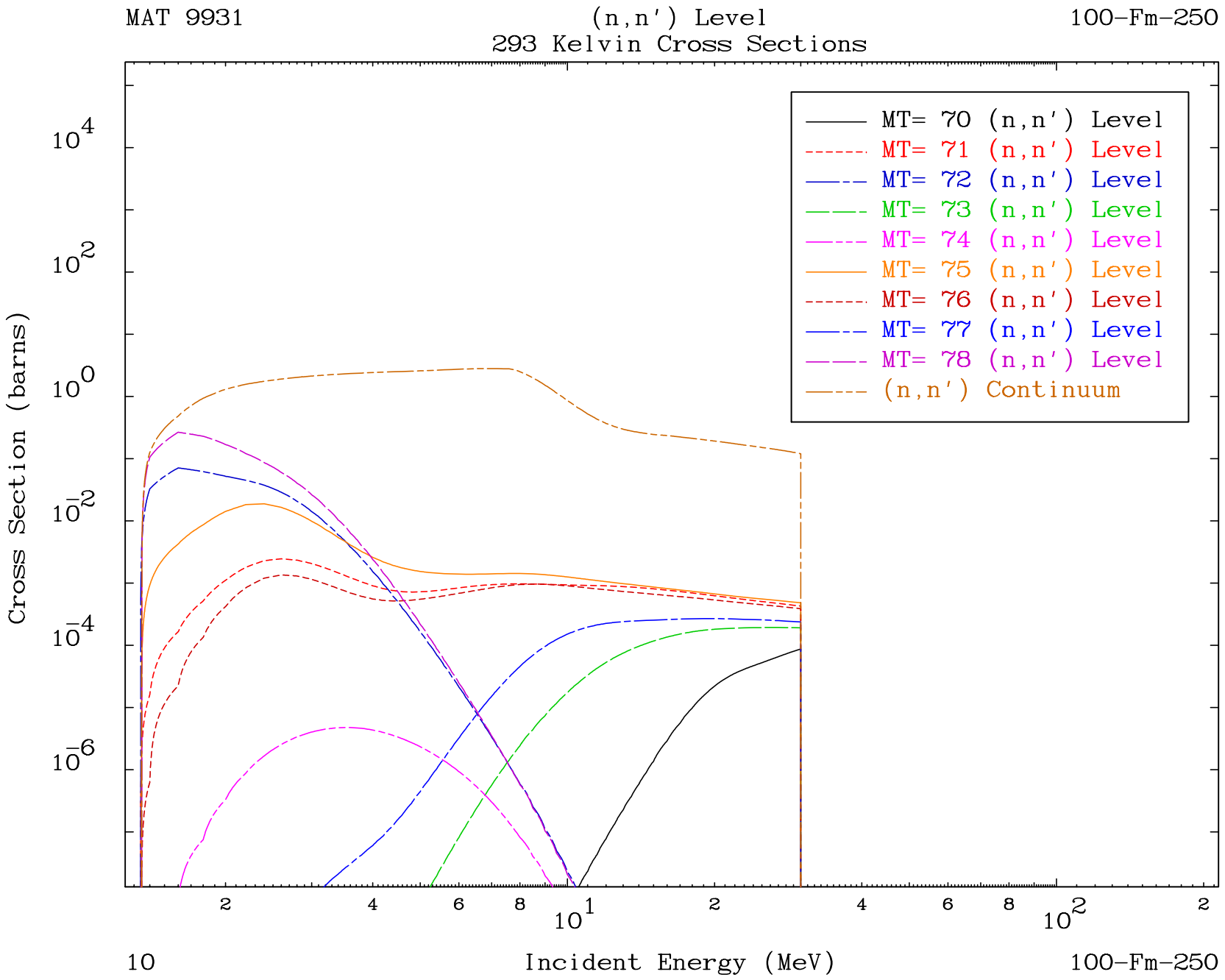

MAT 9931
Charged Particle
293 Kelvin Cross Sections
100-Fm-250
Cross Section (barns)
Incident Energy (MeV)
100-Fm-250
(n,p)
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,2α)
(n,2p)
(n,p) α
(n,p) d
(n,p) t

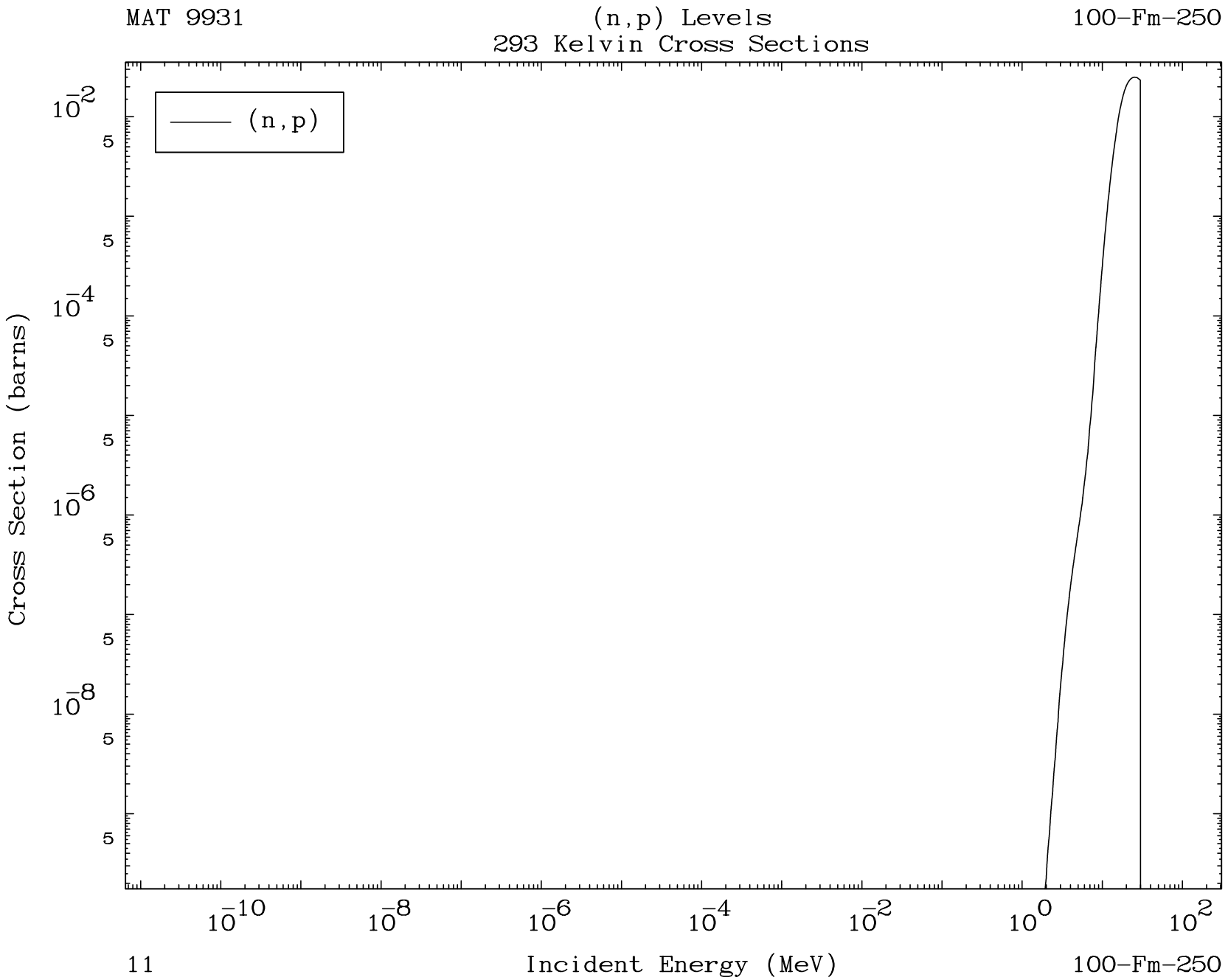

MAT 9931
Particle Production
293 Kelvin Cross Sections
100-Fm-250
Hydrogen Production
Deuterium Production
Tritium Production
He-3 Production
He-4 Production
Cross Section (barns)
Incident Energy (MeV)
6
100-Fm-250

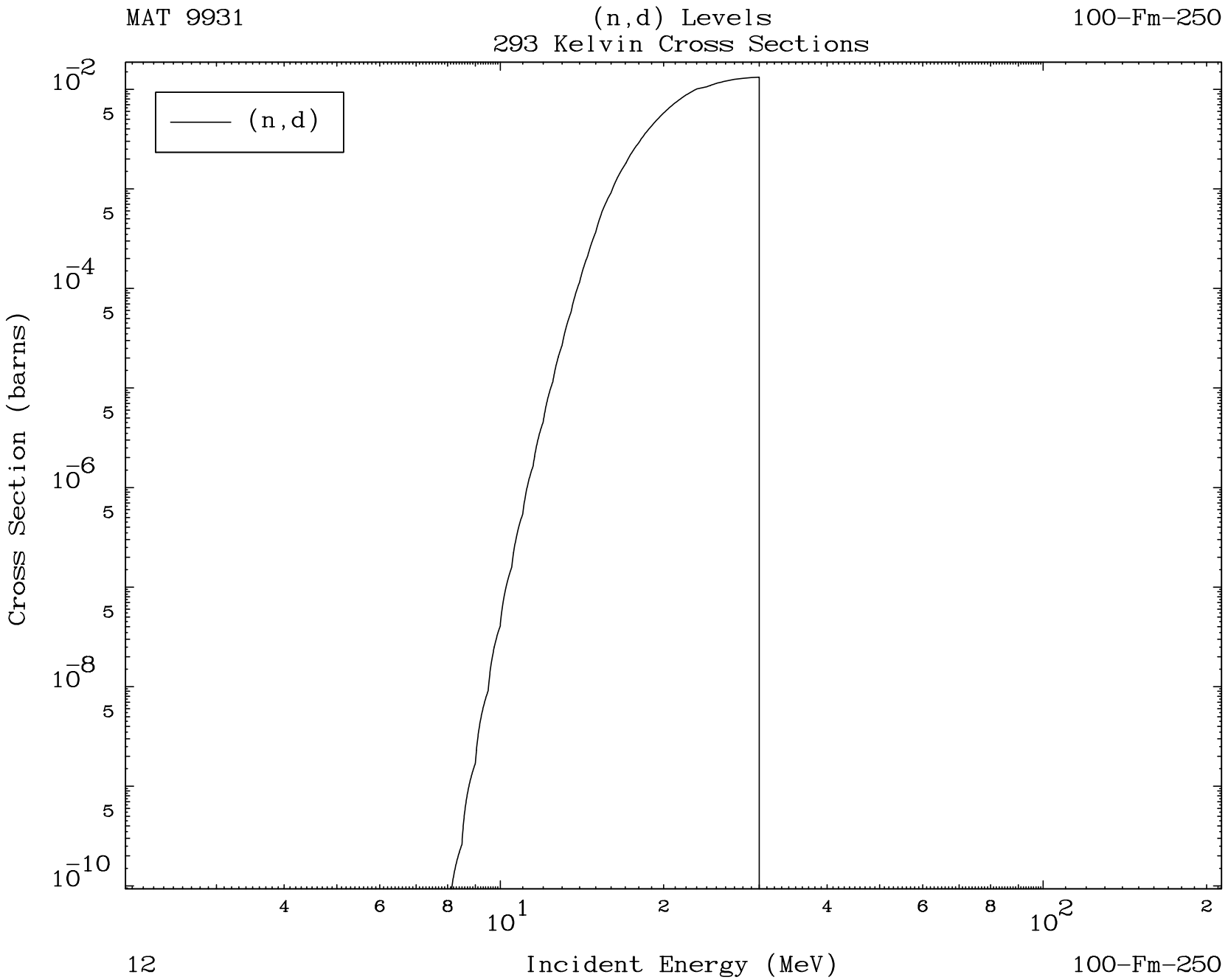

MAT 9931
Fission
100-Fm-250
293 Kelvin Cross Sections
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,3nf) Fourth Chance
Cross Section (barns)
Incident Energy (MeV)
7
100-Fm-250

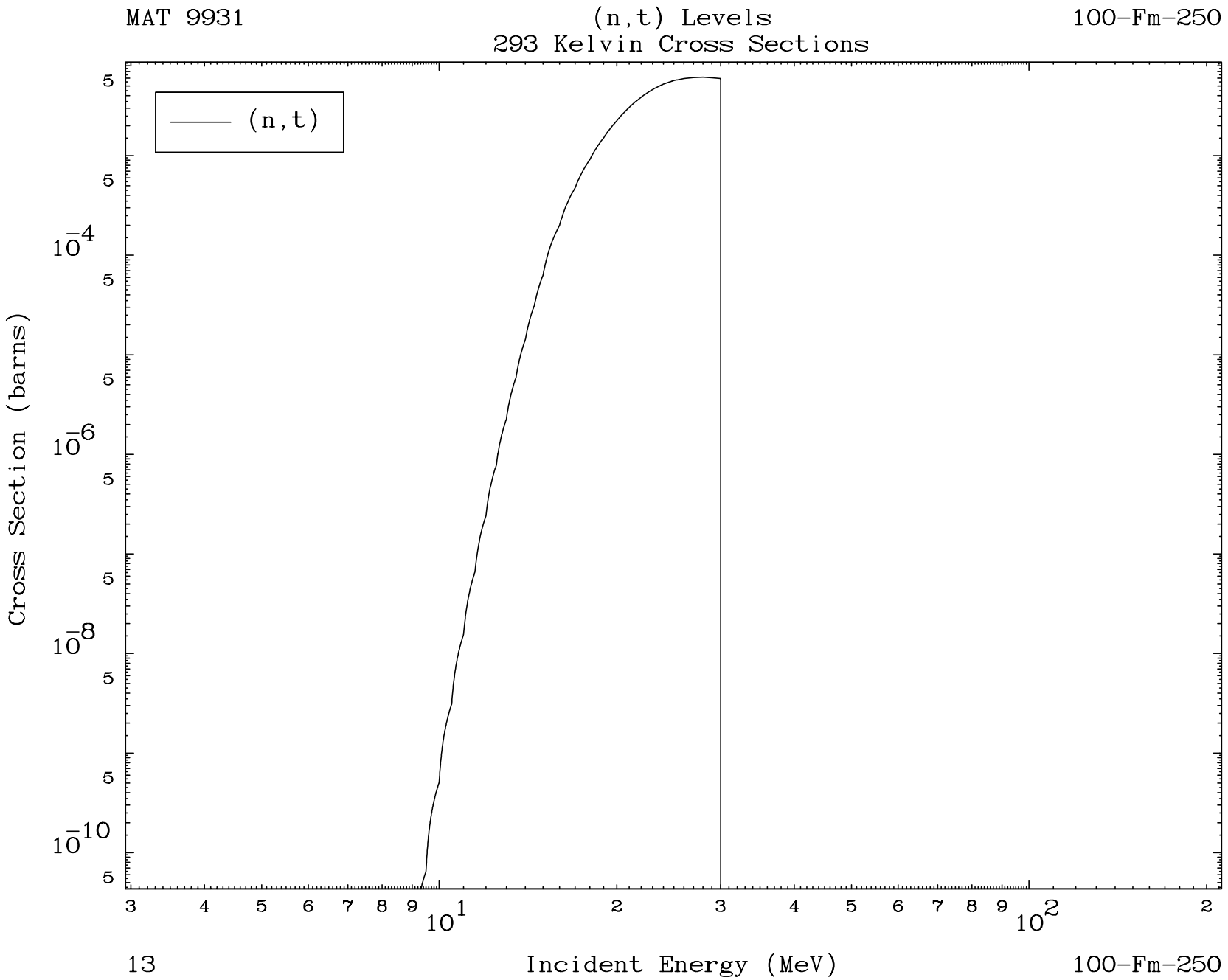

MAT 9931
(n,n') Level
293 Kelvin Cross Sections
100-Fm-250
Inelastic
MT= 51 (n,n') Level
MT= 52 (n,n') Level
MT= 53 (n,n') Level
MT= 54 (n,n') Level
MT= 55 (n,n') Level
MT= 56 (n,n') Level
MT= 58 (n,n') Level
MT= 59 (n,n') Level
Cross Section (barns)
Incident Energy (MeV)
8
100-Fm-250

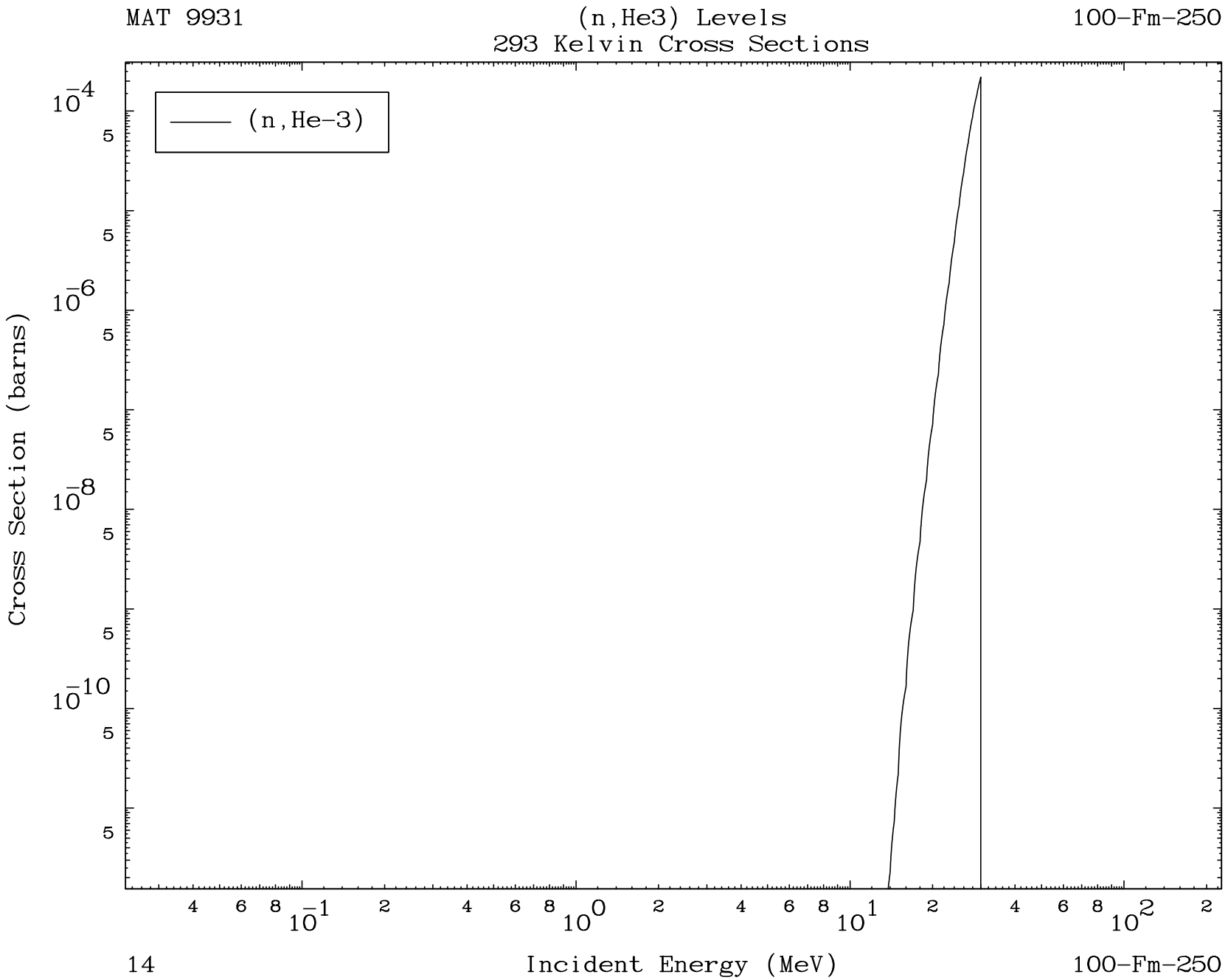

MAT 9931
(n,n') Level
293 Kelvin Cross Sections
100-Fm-250
MT= 60 (n,n') Level
MT= 61 (n,n') Level
MT= 62 (n,n') Level
MT= 63 (n,n') Level
MT= 64 (n,n') Level
MT= 65 (n,n') Level
MT= 66 (n,n') Level
MT= 67 (n,n') Level
MT= 68 (n,n') Level
MT= 69 (n,n') Level
Cross Section (barns)
10^2
10^0
10^-2
10^-4
10^-6
10^-8
2
4
6
8
10^1
2
4
6
8
10^2
2
9
Incident Energy (MeV)
100-Fm-250

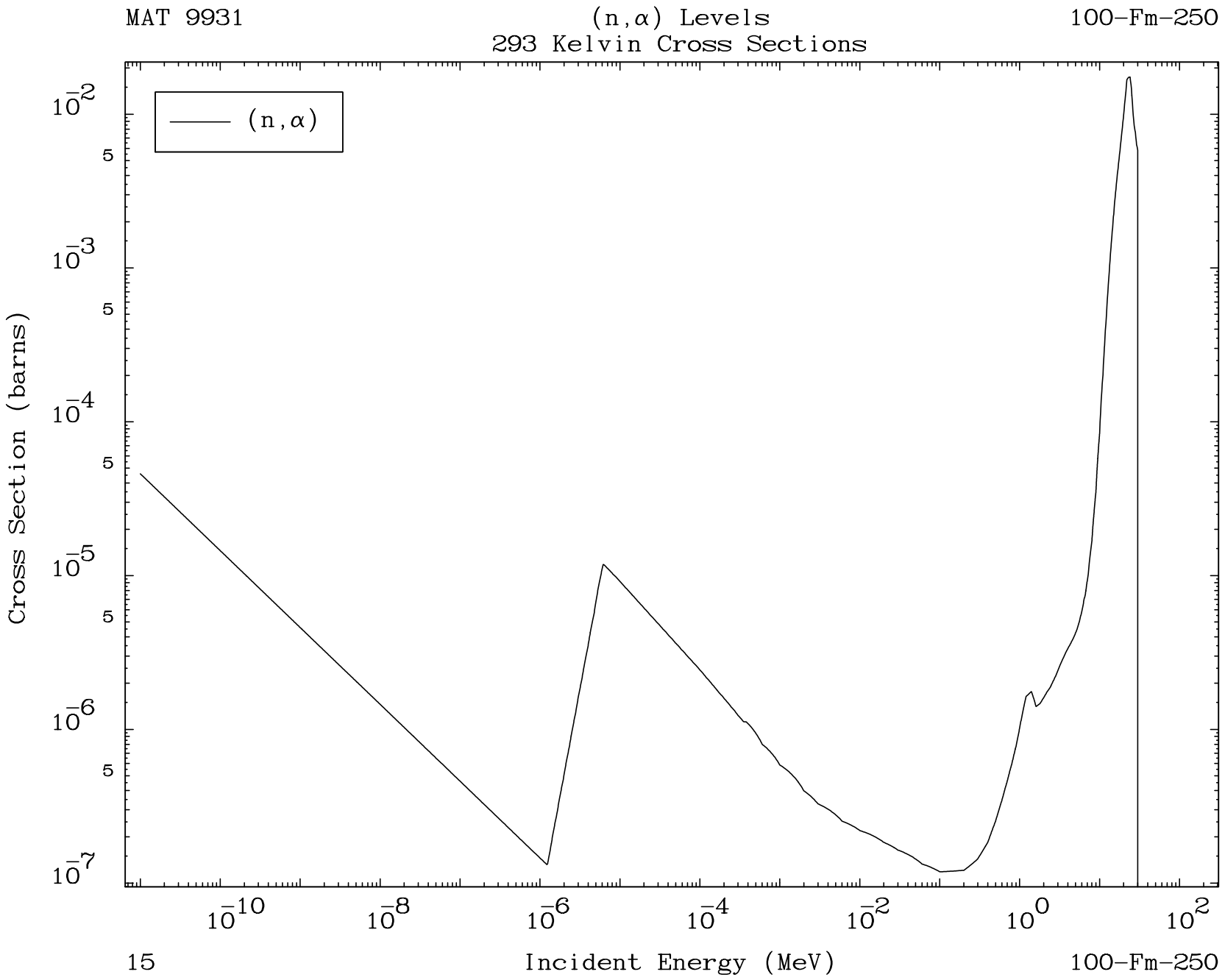

MAT 9931
(n,n') Level
293 Kelvin Cross Sections
100-Fm-250
MT= 70 (n,n') Level
MT= 71 (n,n') Level
MT= 72 (n,n') Level
MT= 73 (n,n') Level
MT= 74 (n,n') Level
MT= 75 (n,n') Level
MT= 76 (n,n') Level
MT= 77 (n,n') Level
MT= 78 (n,n') Level
(n,n') Continuum
Cross Section (barns)
10^4
10^2
10^0
10^-2
10^-4
10^-6
2
4
6
8
10^1
2
4
6
8
10^2
2
10
Incident Energy (MeV)
100-Fm-250

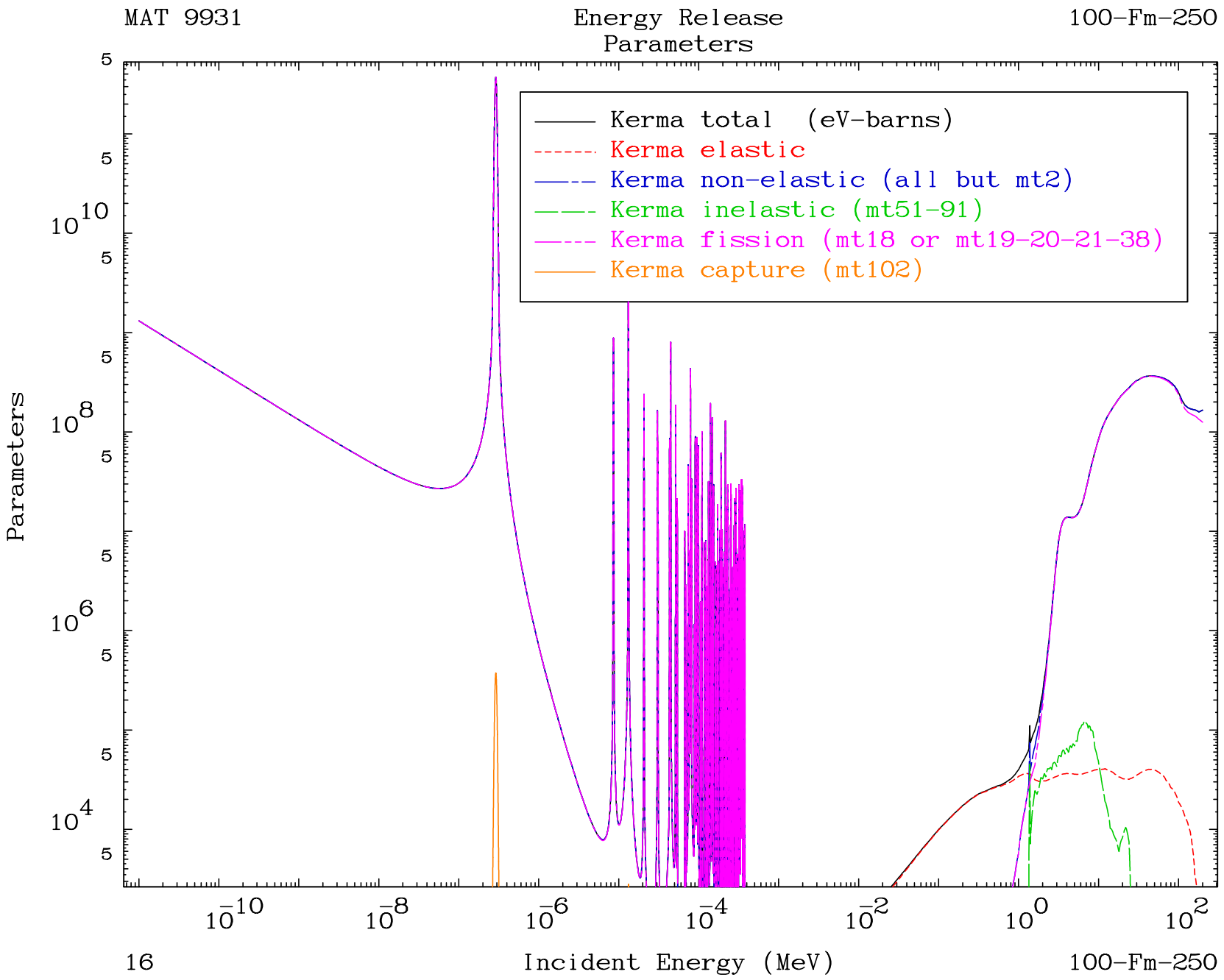

MAT 9931
(n,p) Levels
100-Fm-250
293 Kelvin Cross Sections
(n,p)
Cross Section (barns)
Incident Energy (MeV)
11
100-Fm-250

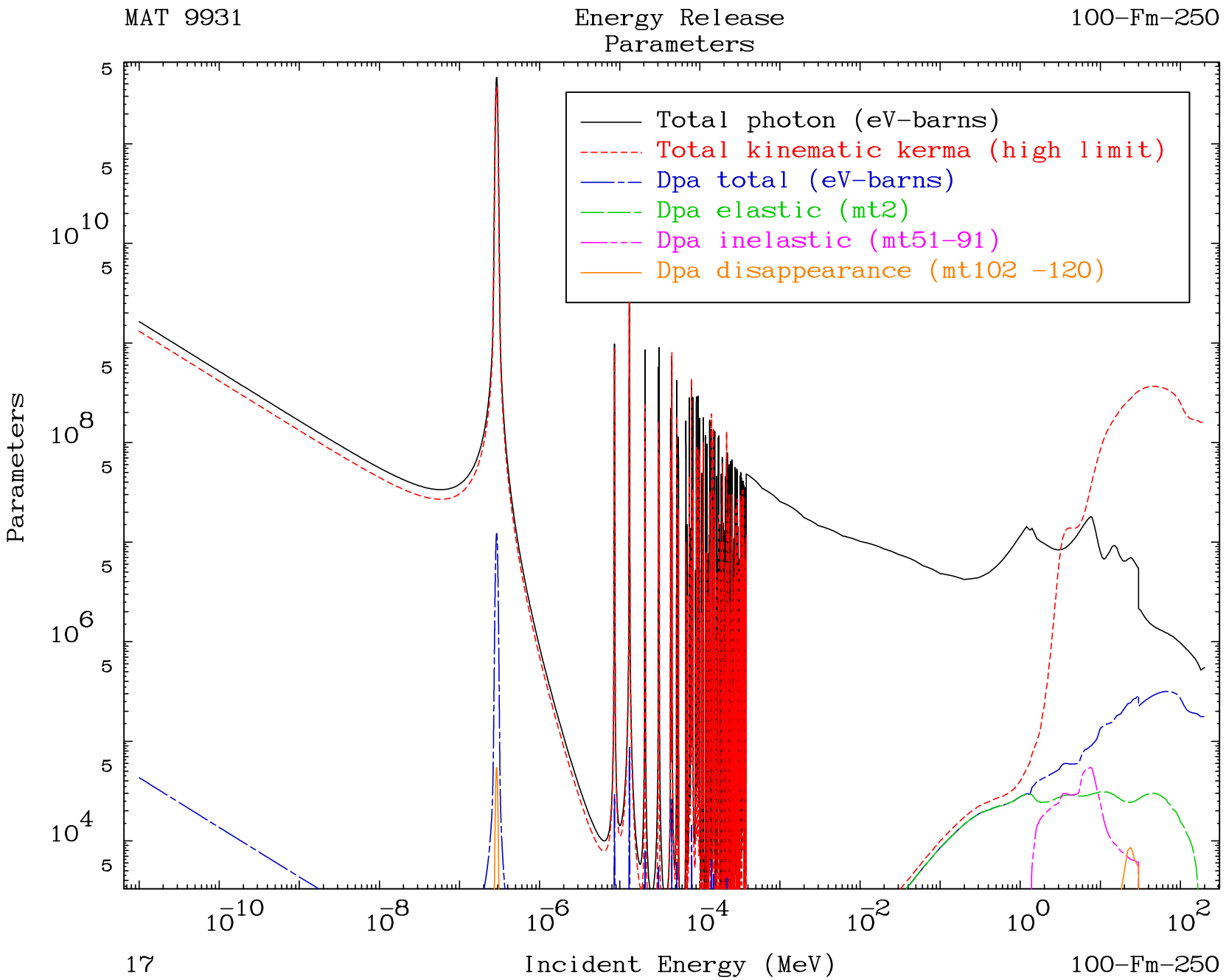

MAT 9931
(n,d) Levels
100-Fm-250
293 Kelvin Cross Sections
(n,d)
Cross Section (barns)
Incident Energy (MeV)
12
100-Fm-250

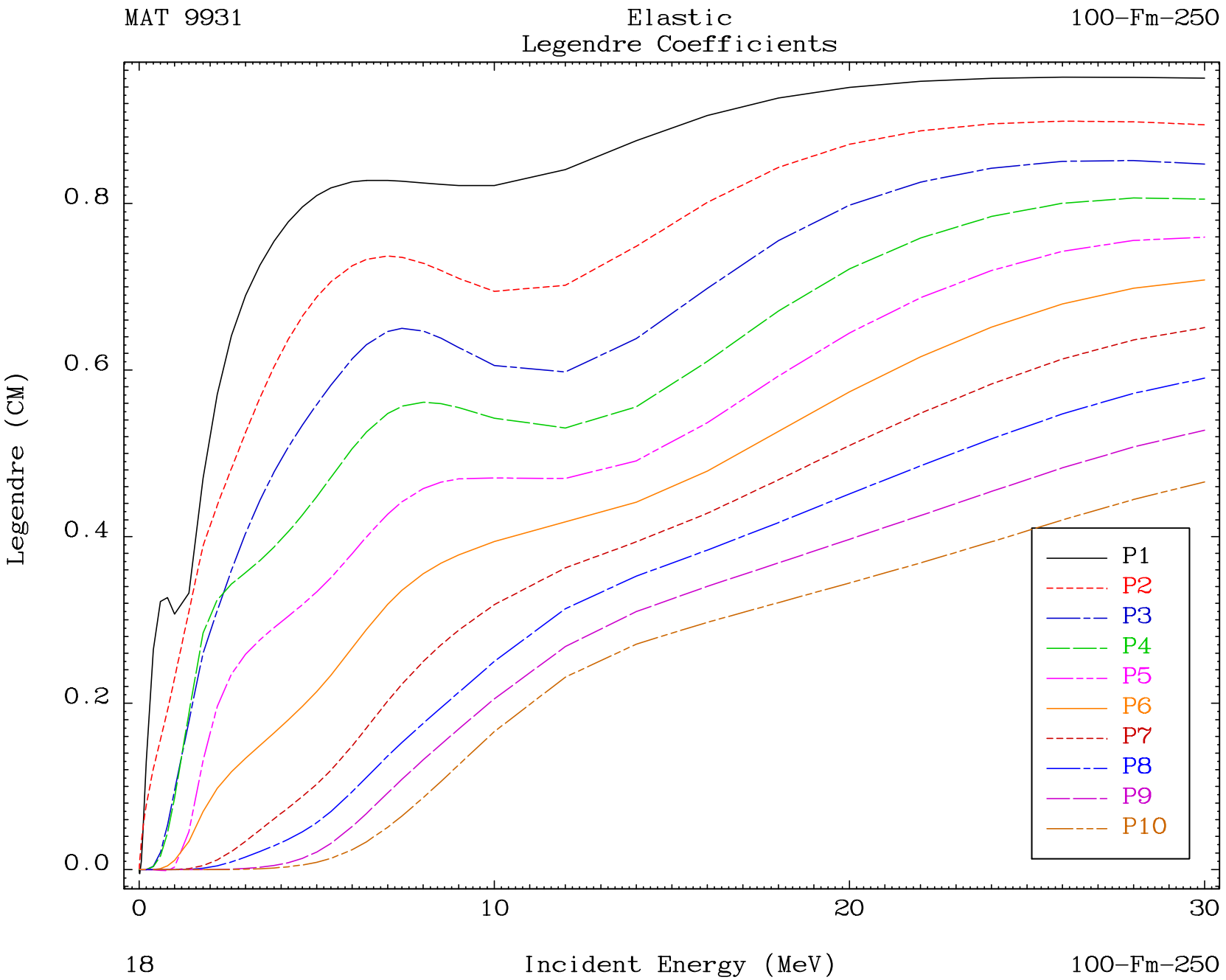

MAT 9931
(n,t) Levels
100-Fm-250
293 Kelvin Cross Sections
(n,t)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
10^1
10^2
Incident Energy (MeV)
13
100-Fm-250

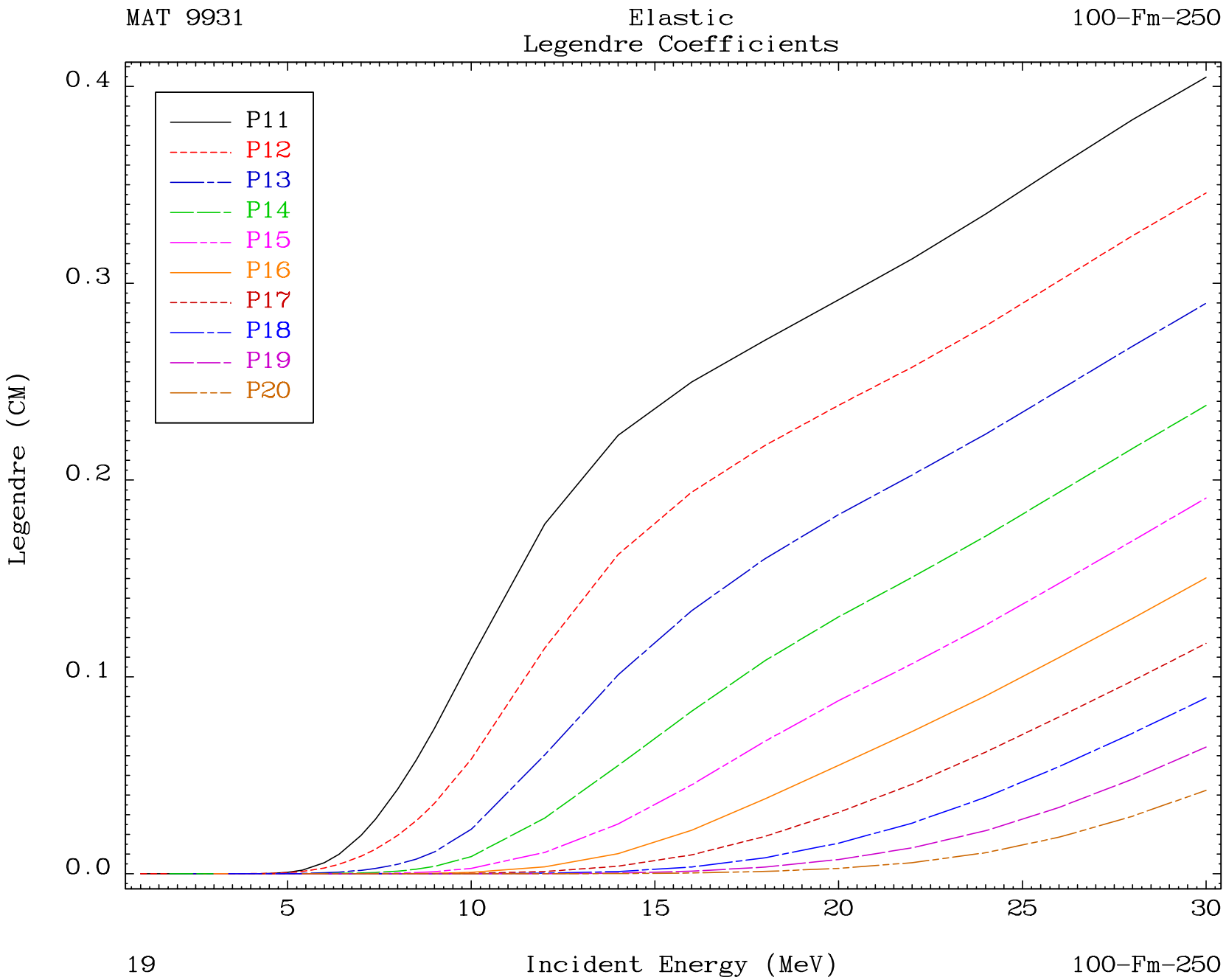

MAT 9931
(n,He3) Levels
100-Fm-250
293 Kelvin Cross Sections
(n,He-3)
Cross Section (barns)
Incident Energy (MeV)
14
100-Fm-250

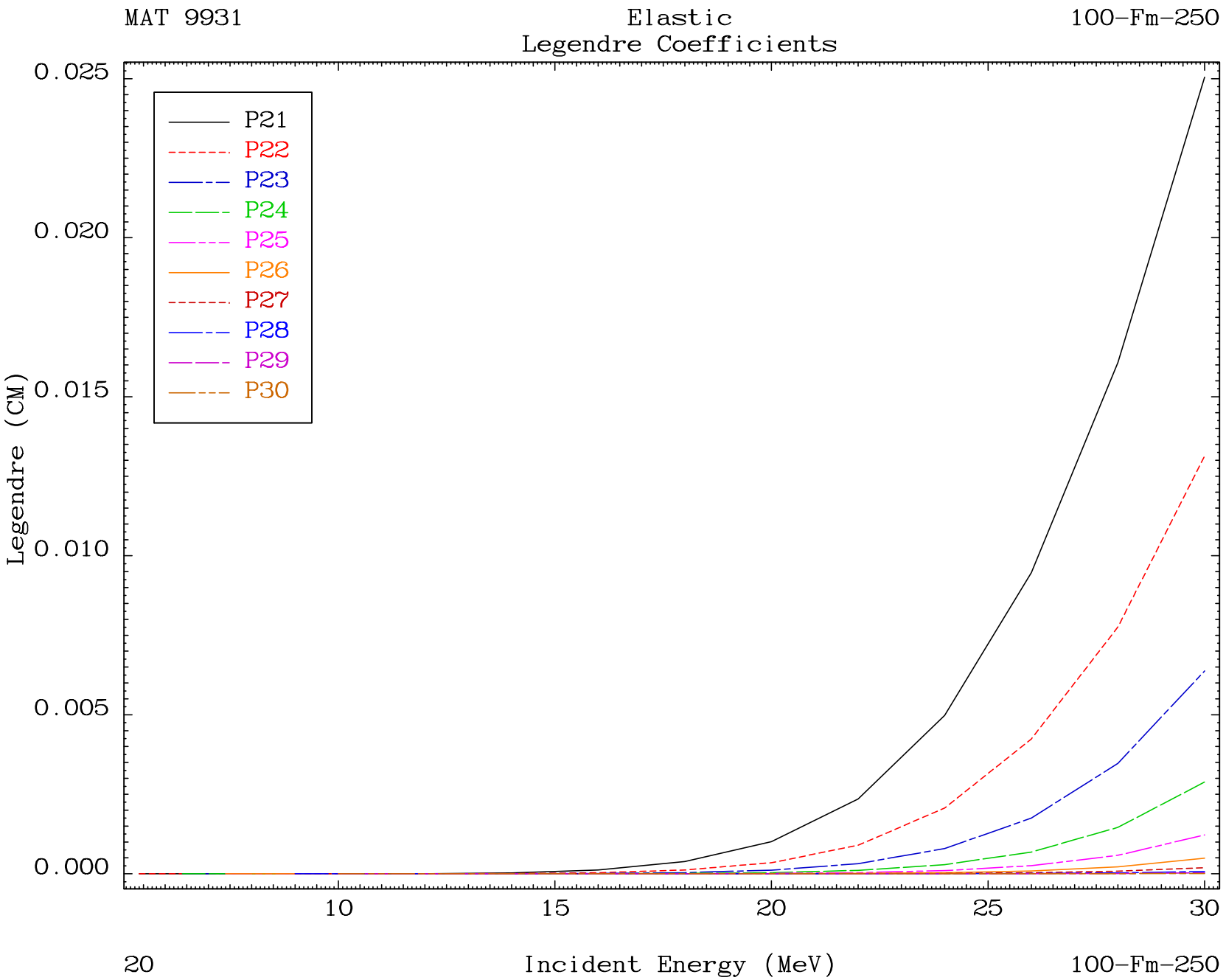

MAT 9931
(n,α) Levels
100-Fm-250
293 Kelvin Cross Sections
(n,α)
Cross Section (barns)
10-2
5
10-3
5
10-4
5
10-5
5
10-6
5
10-7
10-10
10-8
10-6
10-4
10-2
100
102
15
Incident Energy (MeV)
100-Fm-250

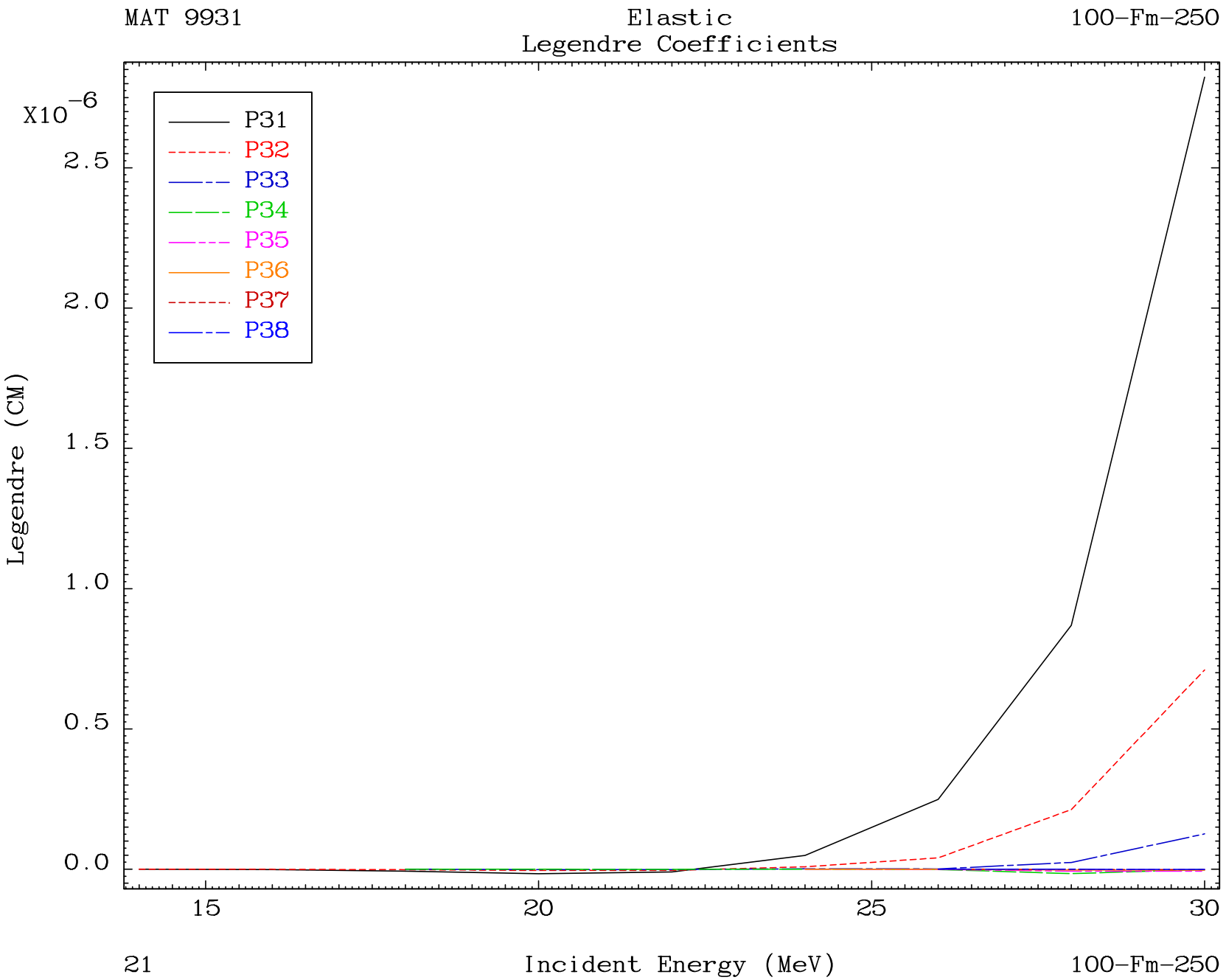

MAT 9931
Energy Release
Parameters
100-Fm-250
Kerma total (eV-barns)
Kerma elastic
Kerma non-elastic (all but mt2)
Kerma inelastic (mt51-91)
Kerma fission (mt18 or mt19-20-21-38)
Kerma capture (mt102)
Parameters
Incident Energy (MeV)
16
100-Fm-250

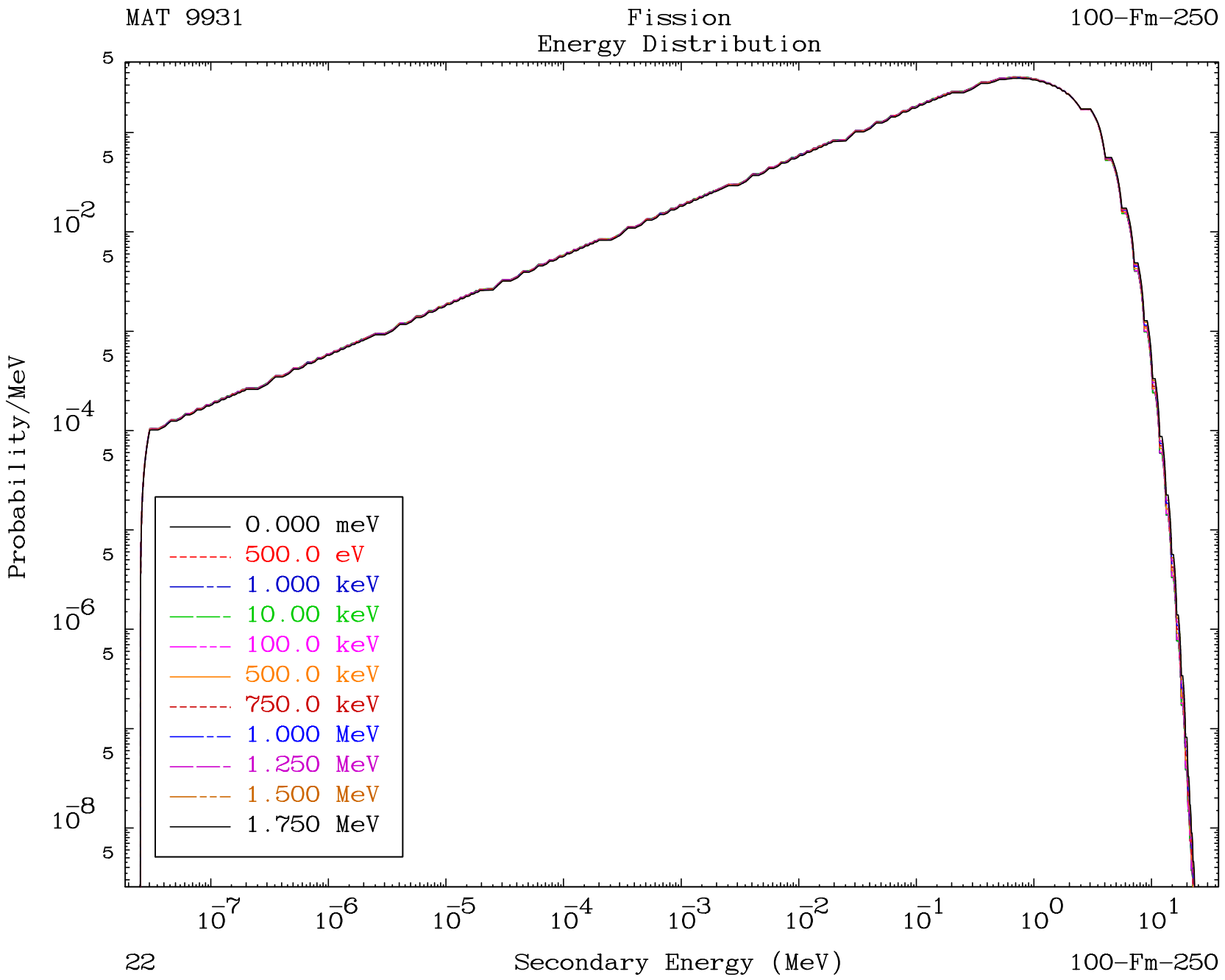

MAT 9931
Energy Release
Parameters
100-Fm-250
Total photon (eV-barns)
Total kinematic kerma (high limit)
Dpa total (eV-barns)
Dpa elastic (mt2)
Dpa inelastic (mt51-91)
Dpa disappearance (mt102 -120)
Parameters
Incident Energy (MeV)
17
100-Fm-250

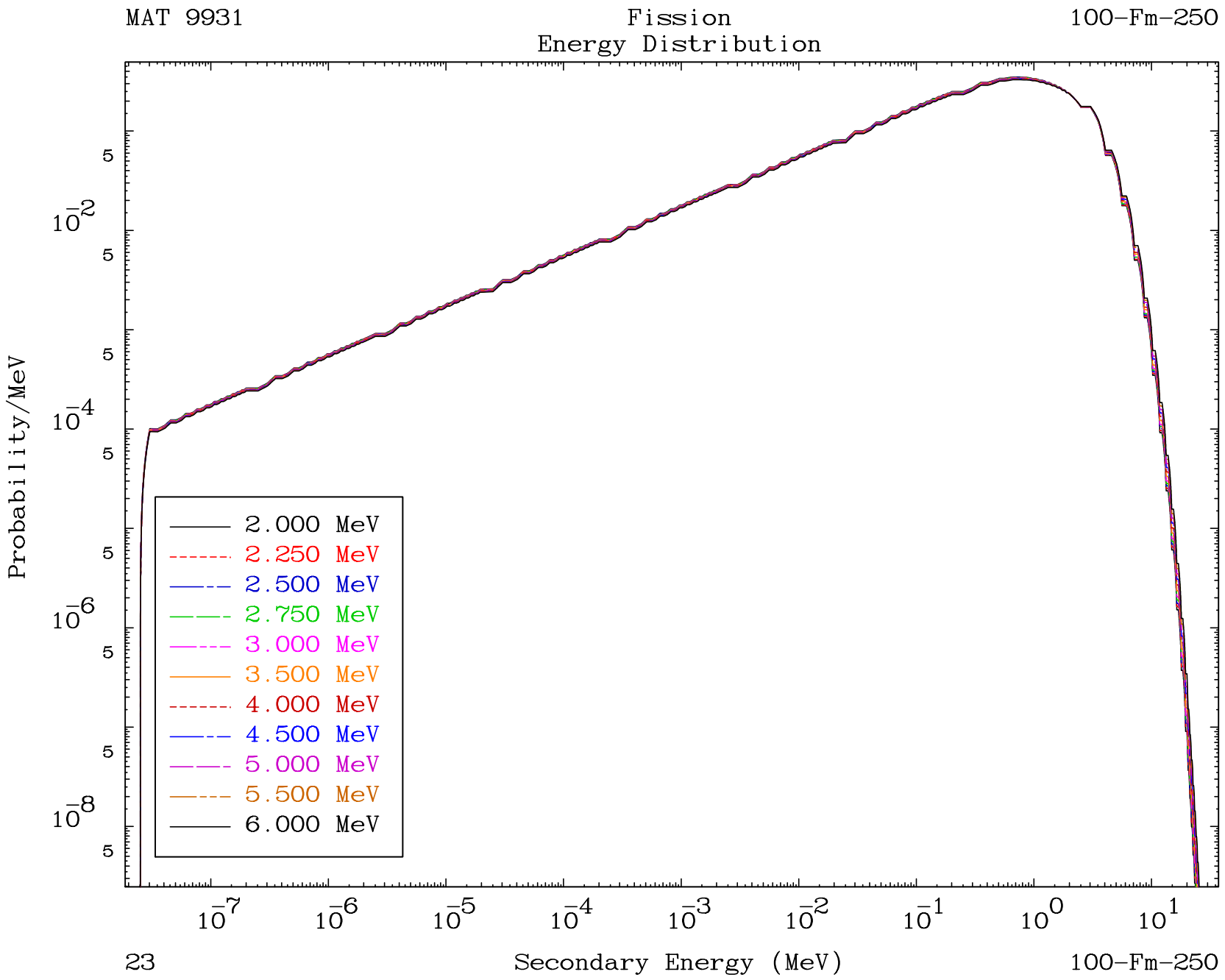

MAT 9931
Elastic
Legendre Coefficients
100-Fm-250
Legendre (CM)
0.0
0.2
0.4
0.6
0.8
0
10
20
30
18
Incident Energy (MeV)
100-Fm-250
P1
P2
P3
P4
P5
P6
P7
P8
P9
P10

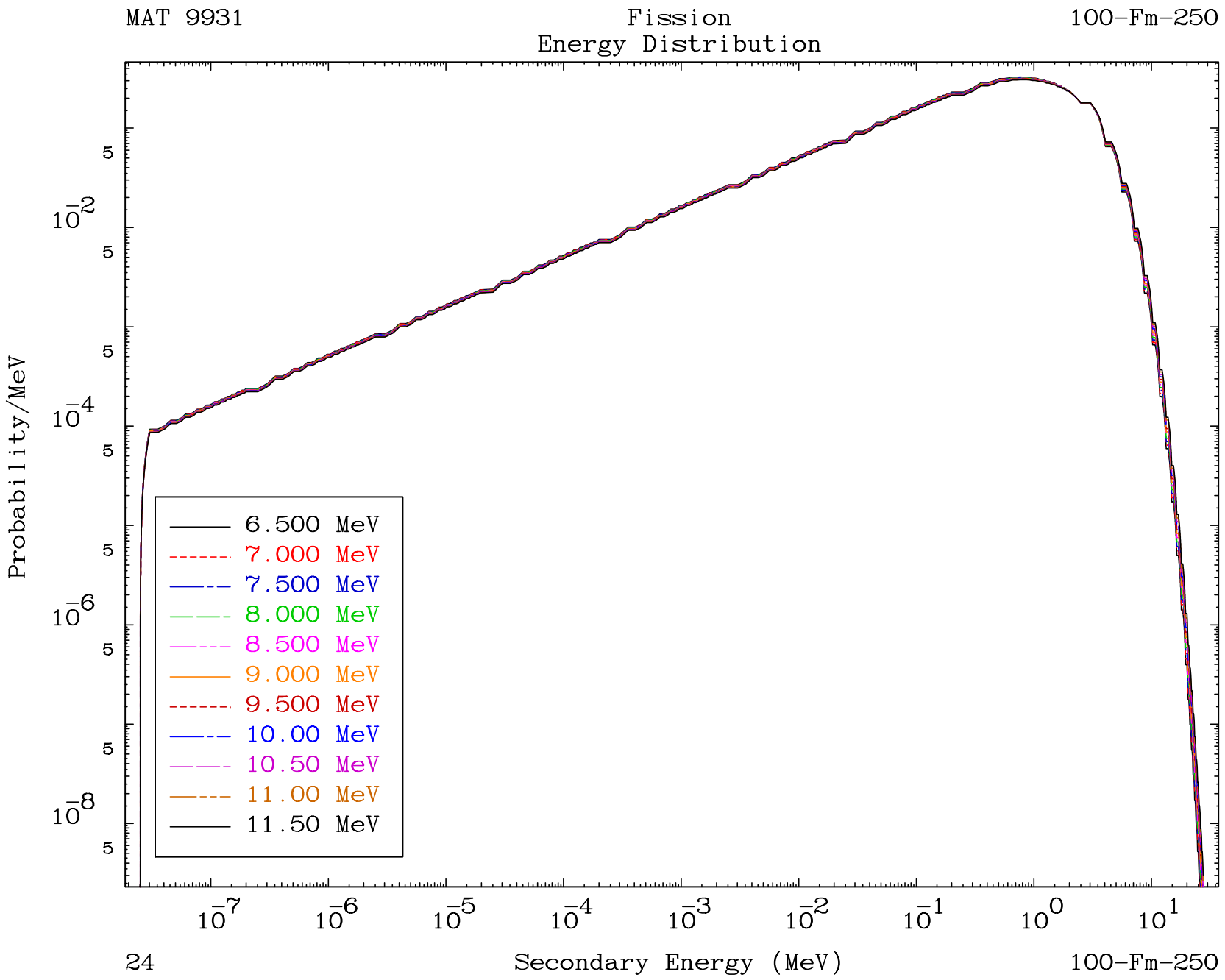

MAT 9931
Elastic
Legendre Coefficients
100-Fm-250
P11
P12
P13
P14
P15
P16
P17
P18
P19
P20
Legendre (CM)
0.4
0.3
0.2
0.1
0.0
5
10
15
20
25
30
19
Incident Energy (MeV)
100-Fm-250

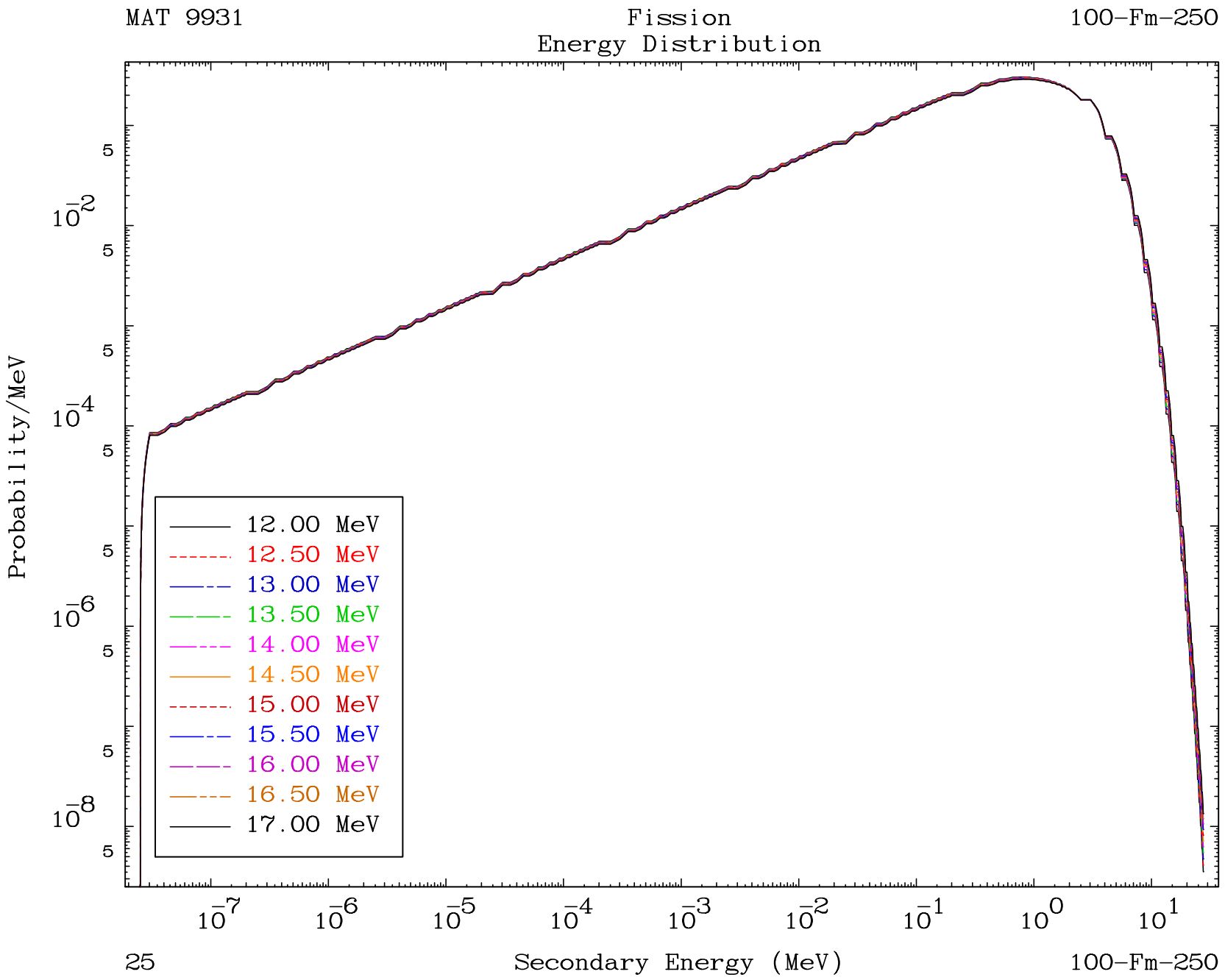

MAT 9931
Elastic
Legendre Coefficients
100-Fm-250
P21
P22
P23
P24
P25
P26
P27
P28
P29
P30
0.025
0.020
0.015
0.010
0.005
0.000
Legendre (CM)
10
15
20
25
30
20
Incident Energy (MeV)
100-Fm-250

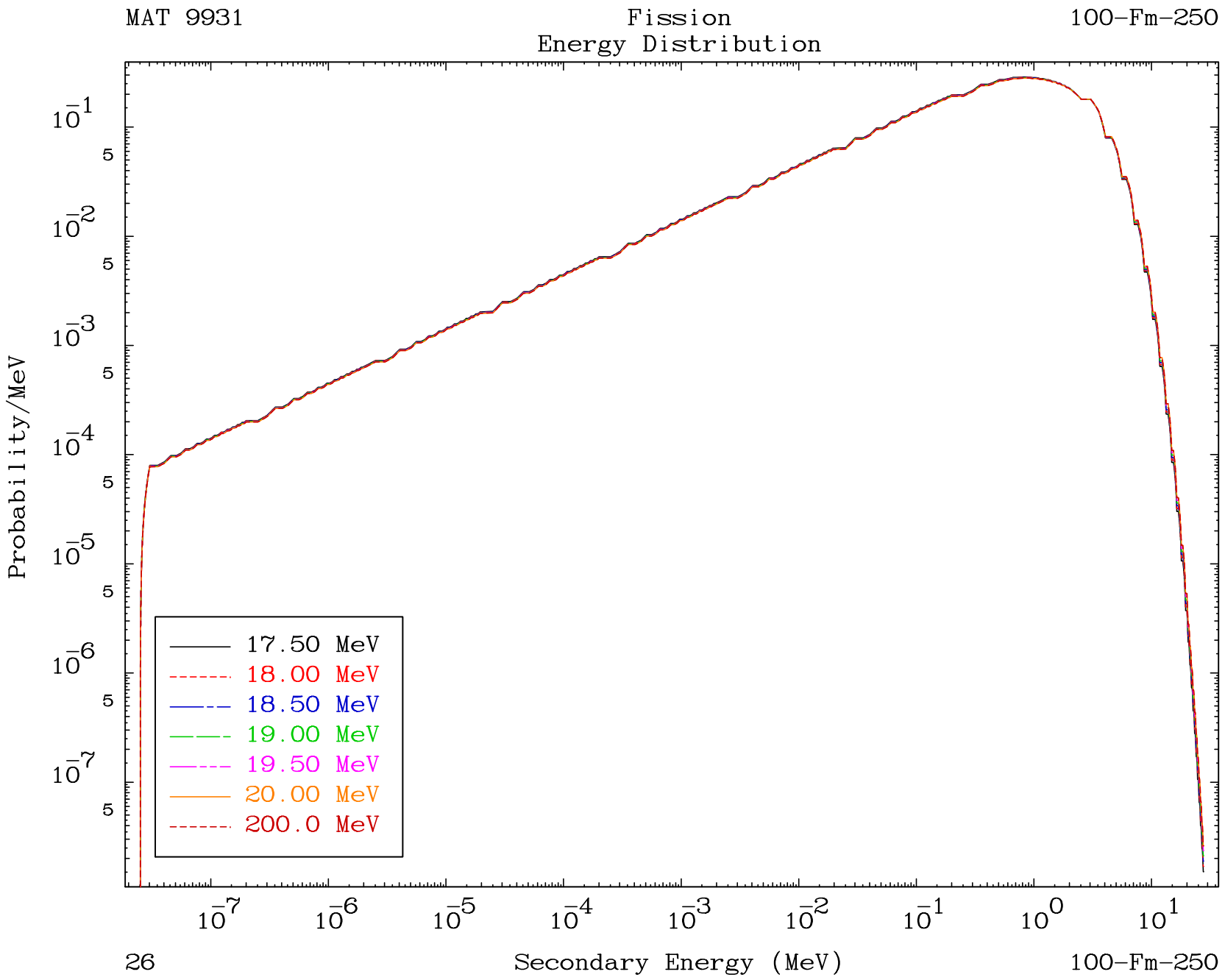

MAT 9931
Elastic
Legendre Coefficients
100-Fm-250
X10 -6
2.5
2.0
1.5
1.0
0.5
0.0
Legendre (CM)
P31
P32
P33
P34
P35
P36
P37
P38
15
20
25
30
21
Incident Energy (MeV)
100-Fm-250

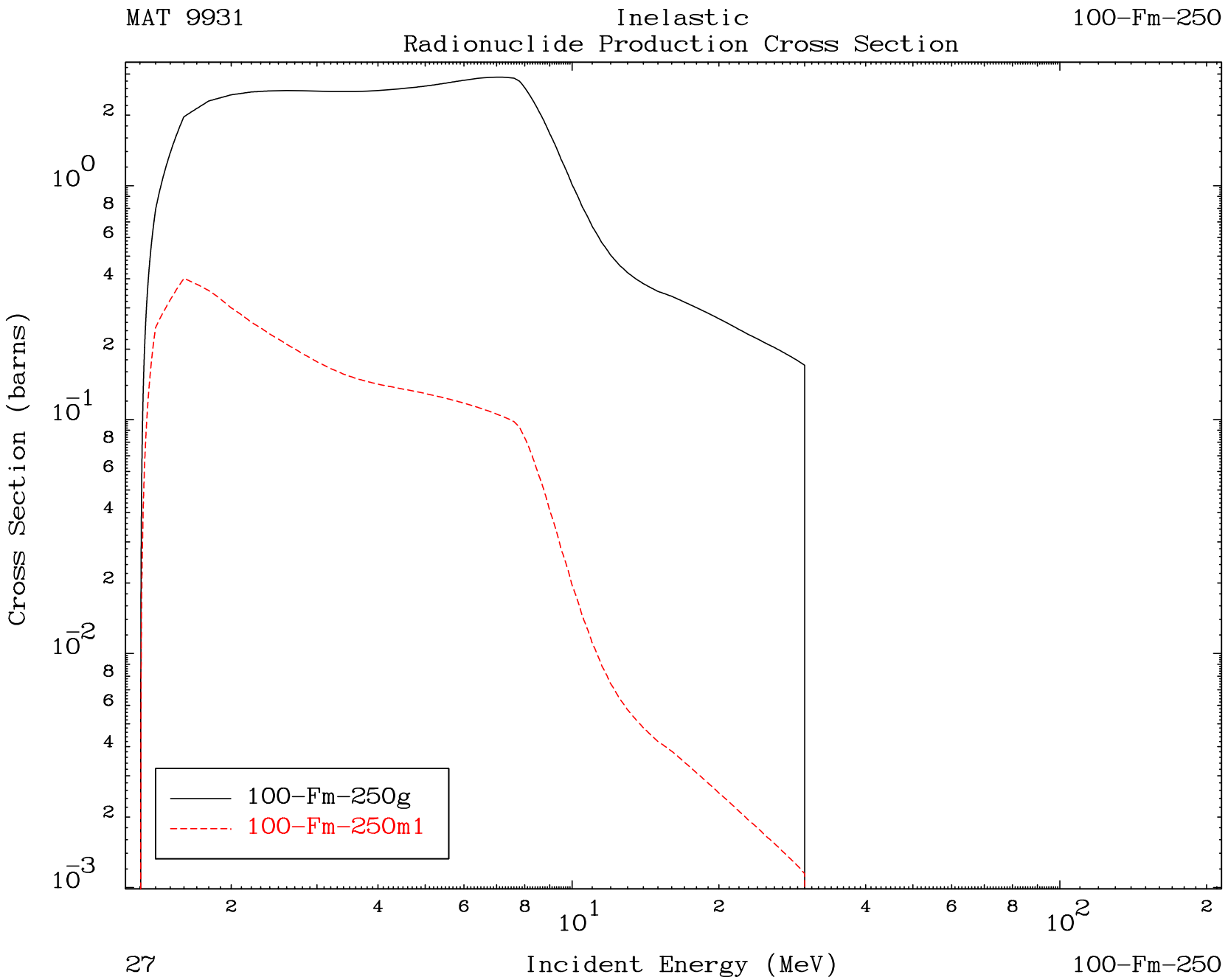

MAT 9931
Fission
Energy Distribution
100-Fm-250
Probability/MeV
0.000 meV
500.0 eV
1.000 keV
10.00 keV
100.0 keV
500.0 keV
750.0 keV
1.000 MeV
1.250 MeV
1.500 MeV
1.750 MeV
Secondary Energy (MeV)
22
100-Fm-250

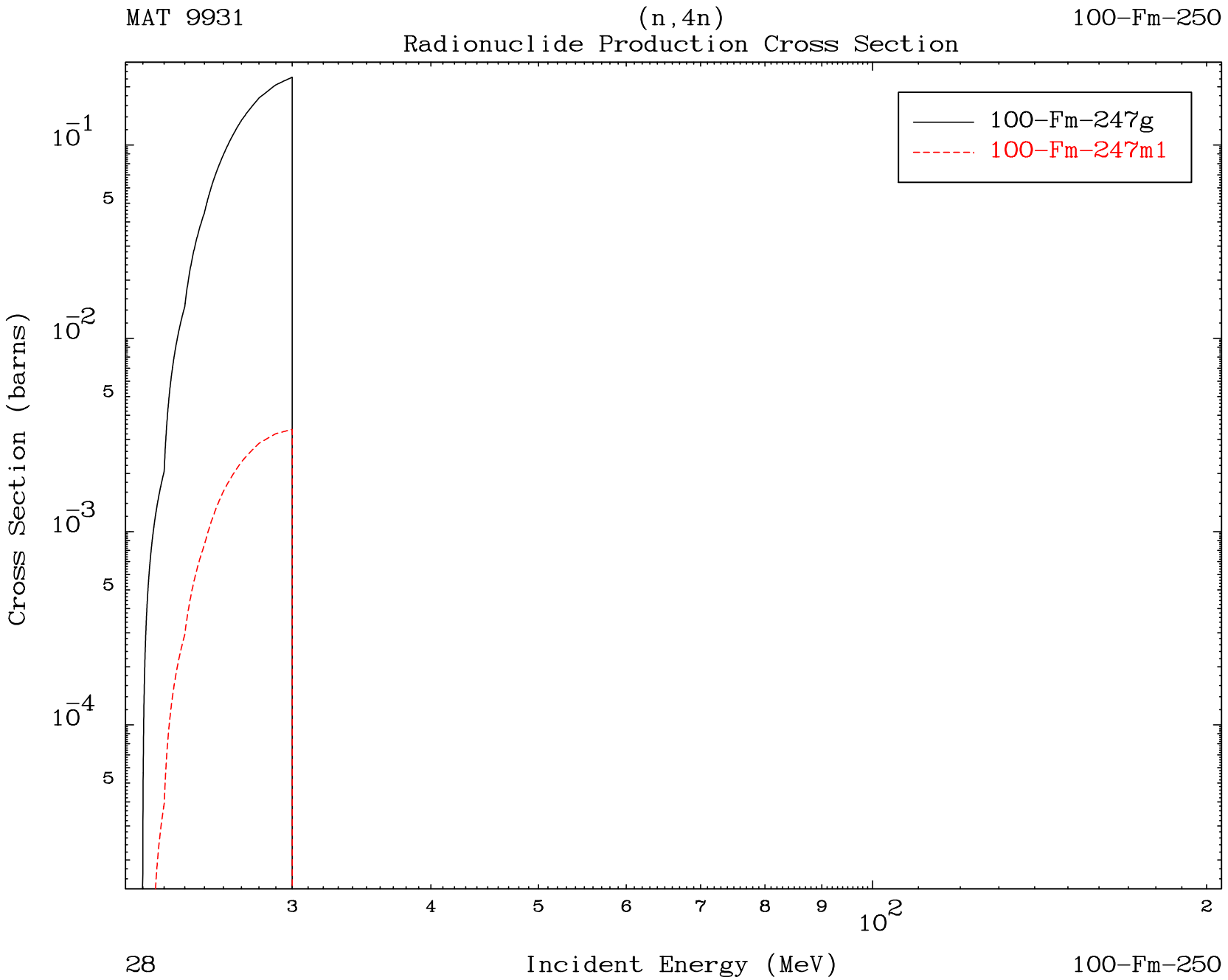

MAT 9931
Fission
Energy Distribution
100-Fm-250
Probability/MeV
2.000 MeV
2.250 MeV
2.500 MeV
2.750 MeV
3.000 MeV
3.500 MeV
4.000 MeV
4.500 MeV
5.000 MeV
5.500 MeV
6.000 MeV
Secondary Energy (MeV)
23
100-Fm-250

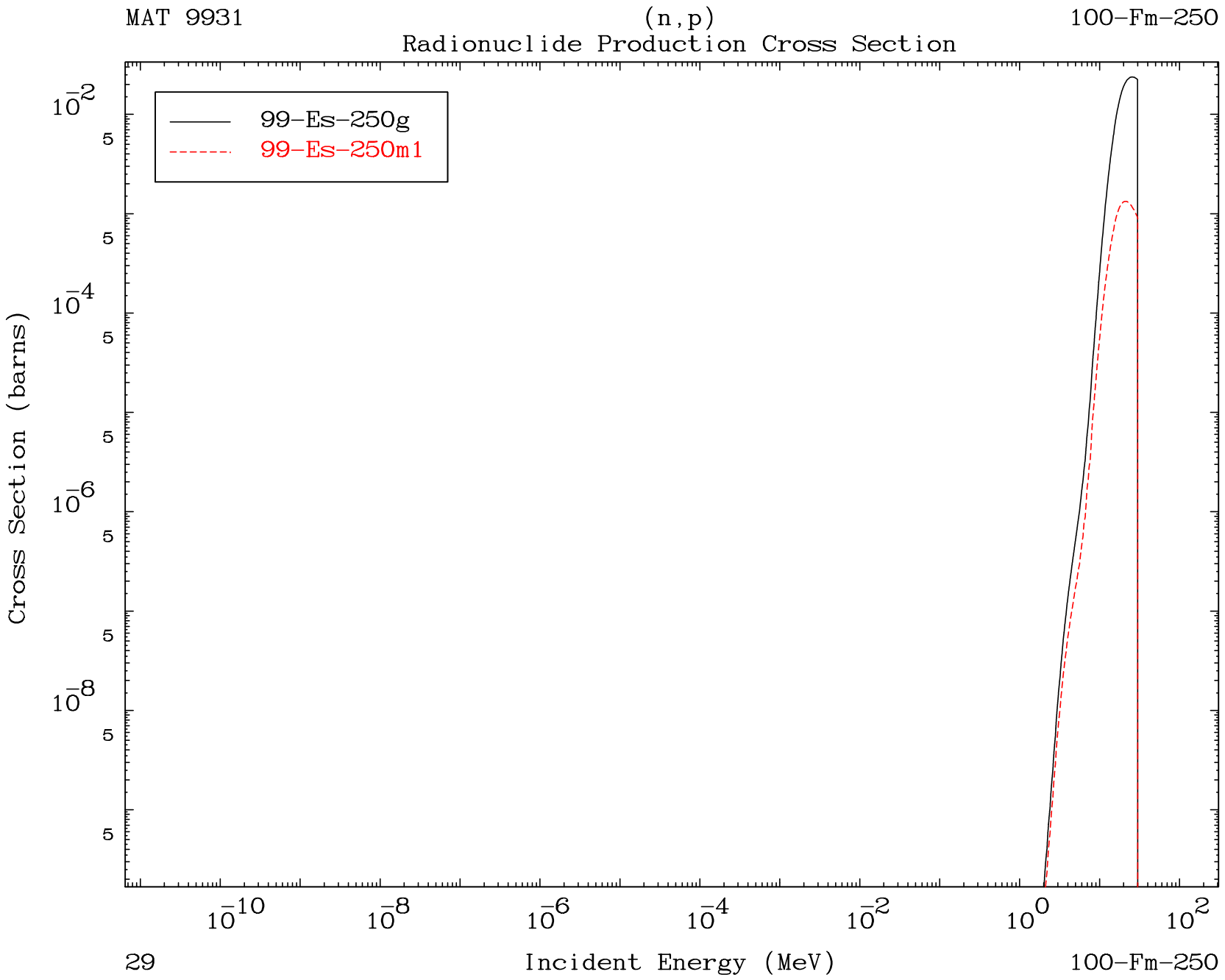

MAT 9931
Fission
Energy Distribution
100-Fm-250
Probability/MeV
6.500 MeV
7.000 MeV
7.500 MeV
8.000 MeV
8.500 MeV
9.000 MeV
9.500 MeV
10.00 MeV
10.50 MeV
11.00 MeV
11.50 MeV
Secondary Energy (MeV)
24
100-Fm-250


MAT 9931
Fission
Energy Distribution
100-Fm-250
Probability/MeV
12.00 MeV
12.50 MeV
13.00 MeV
13.50 MeV
14.00 MeV
14.50 MeV
15.00 MeV
15.50 MeV
16.00 MeV
16.50 MeV
17.00 MeV
Secondary Energy (MeV)
25
100-Fm-250


MAT 9931
Fission
Energy Distribution
100-Fm-250
Probability/MeV
17.50 MeV
18.00 MeV
18.50 MeV
19.00 MeV
19.50 MeV
20.00 MeV
200.0 MeV
Secondary Energy (MeV)
26
100-Fm-250


MAT 9931
Inelastic
100-Fm-250
Radionuclide Production Cross Section
Cross Section (barns)
Incident Energy (MeV)
100-Fm-250g
100-Fm-250m1
27
100-Fm-250


MAT 9931
(n,4n)
100-Fm-250
Radionuclide Production Cross Section
100-Fm-247g
100-Fm-247m1
Cross Section (barns)
10-1
5
10-2
5
10-3
5
10-4
5
3
4
5
6
7
8
9
10 2
2
28
Incident Energy (MeV)
100-Fm-250


MAT 9931
(n,p)
100-Fm-250
Radionuclide Production Cross Section
99-Es-250g
99-Es-250m1
Cross Section (barns)
Incident Energy (MeV)
29
100-Fm-250