

Program EVALPLOT
(Version 2017-1)

by

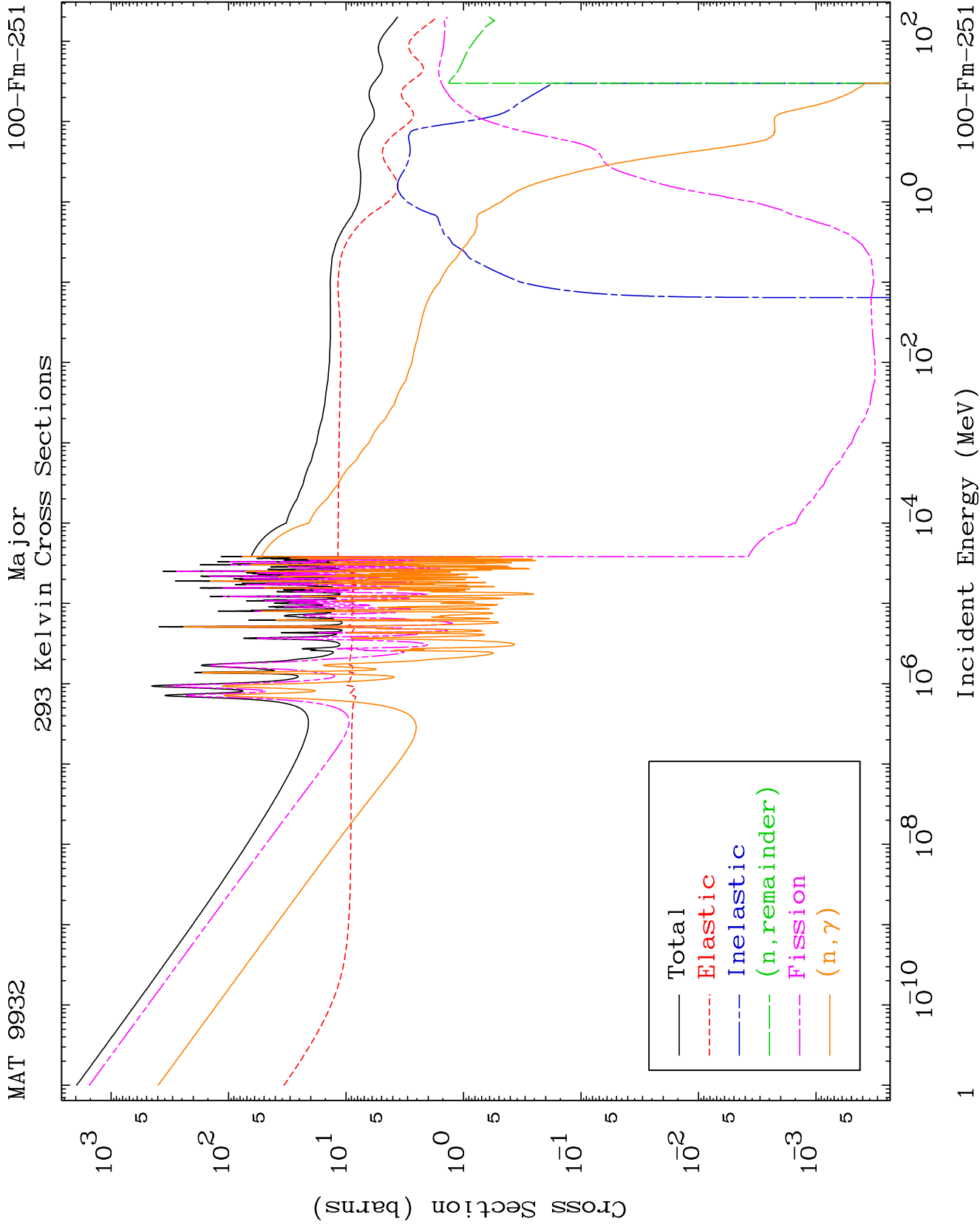
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
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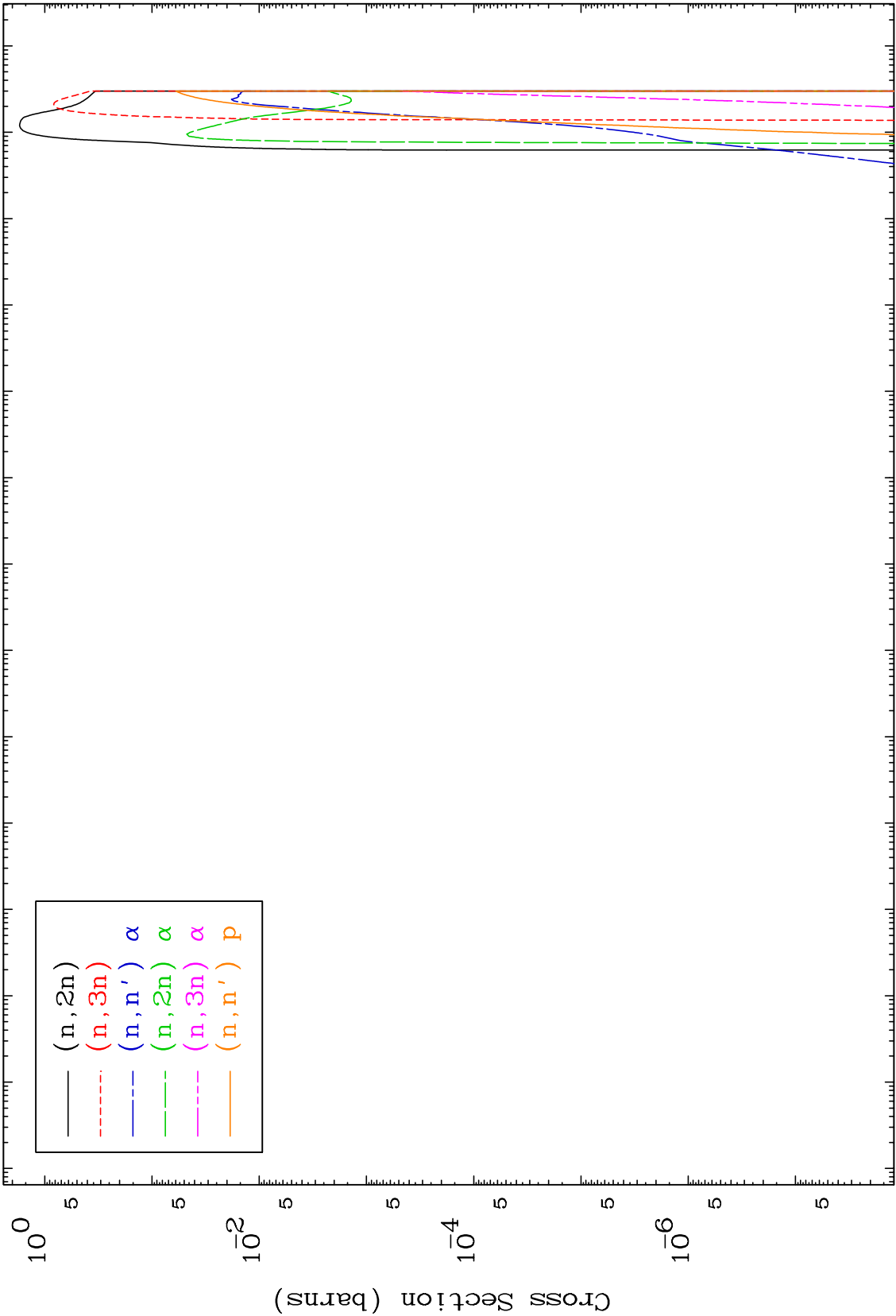
Press Mouse Button to Start



MAT 9932

Neutron Production
293 Kelvin Cross Sections

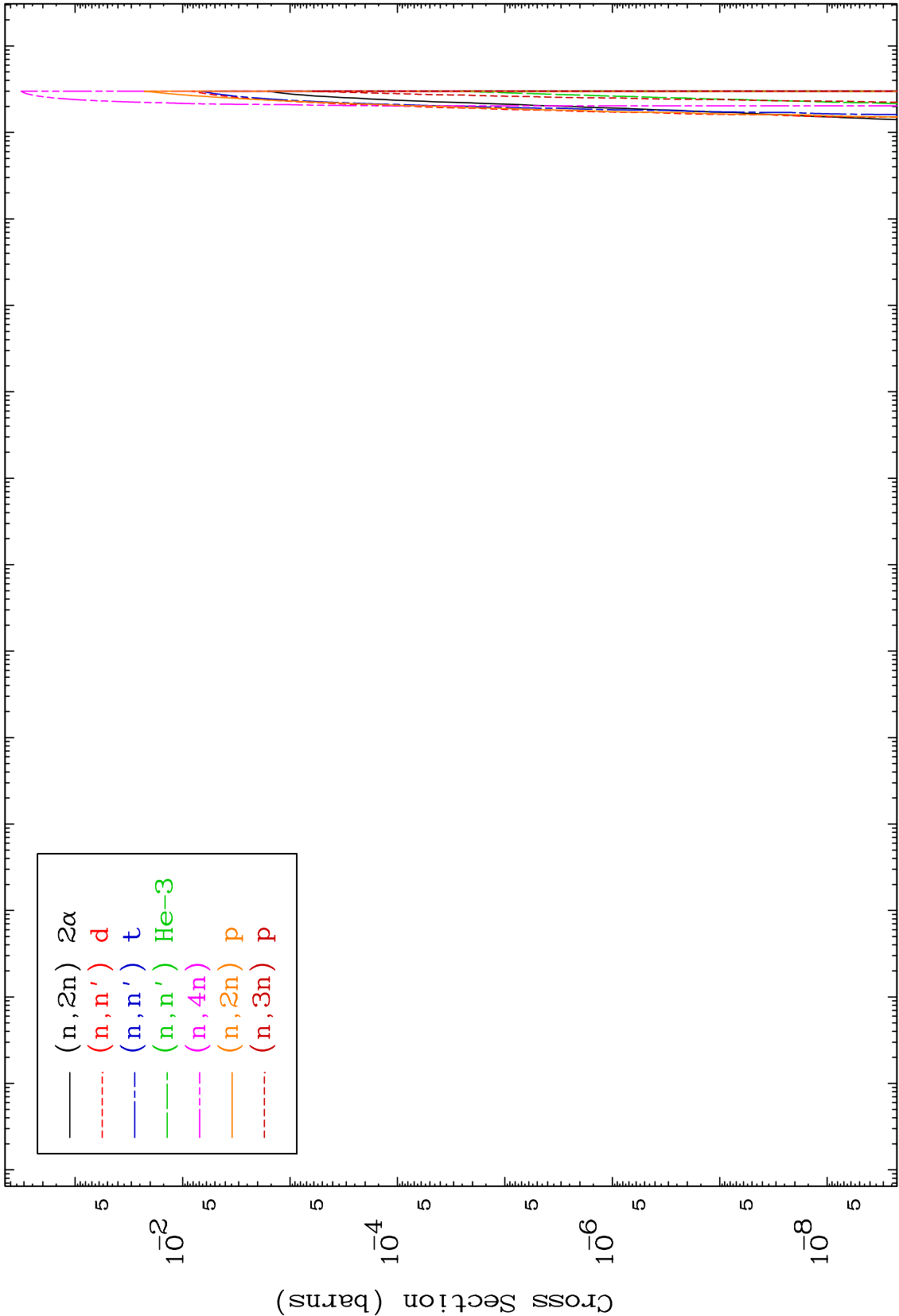
100-Fm-251

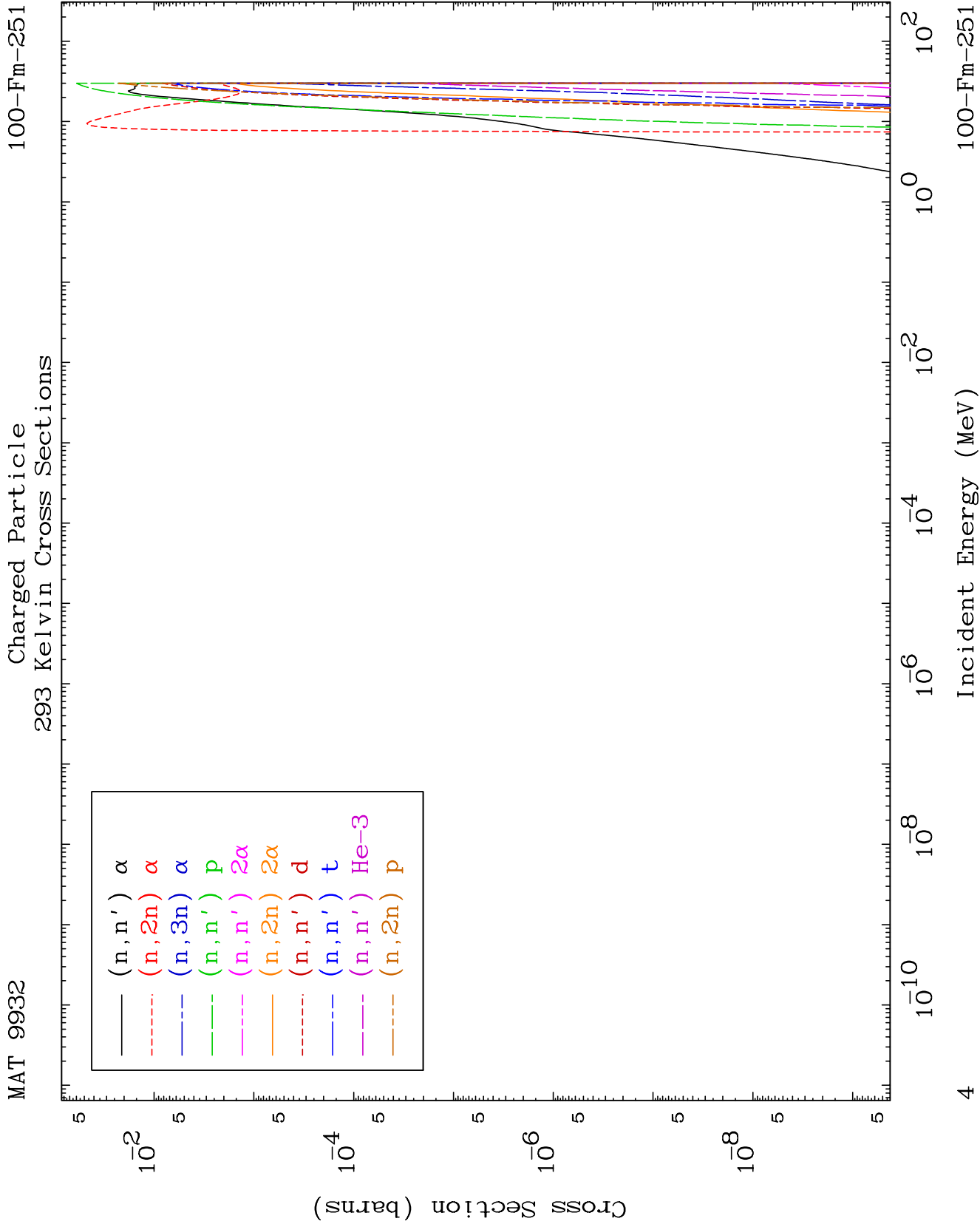


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Neutron Production
293 Kelvin Cross Sections

100-Fm-251

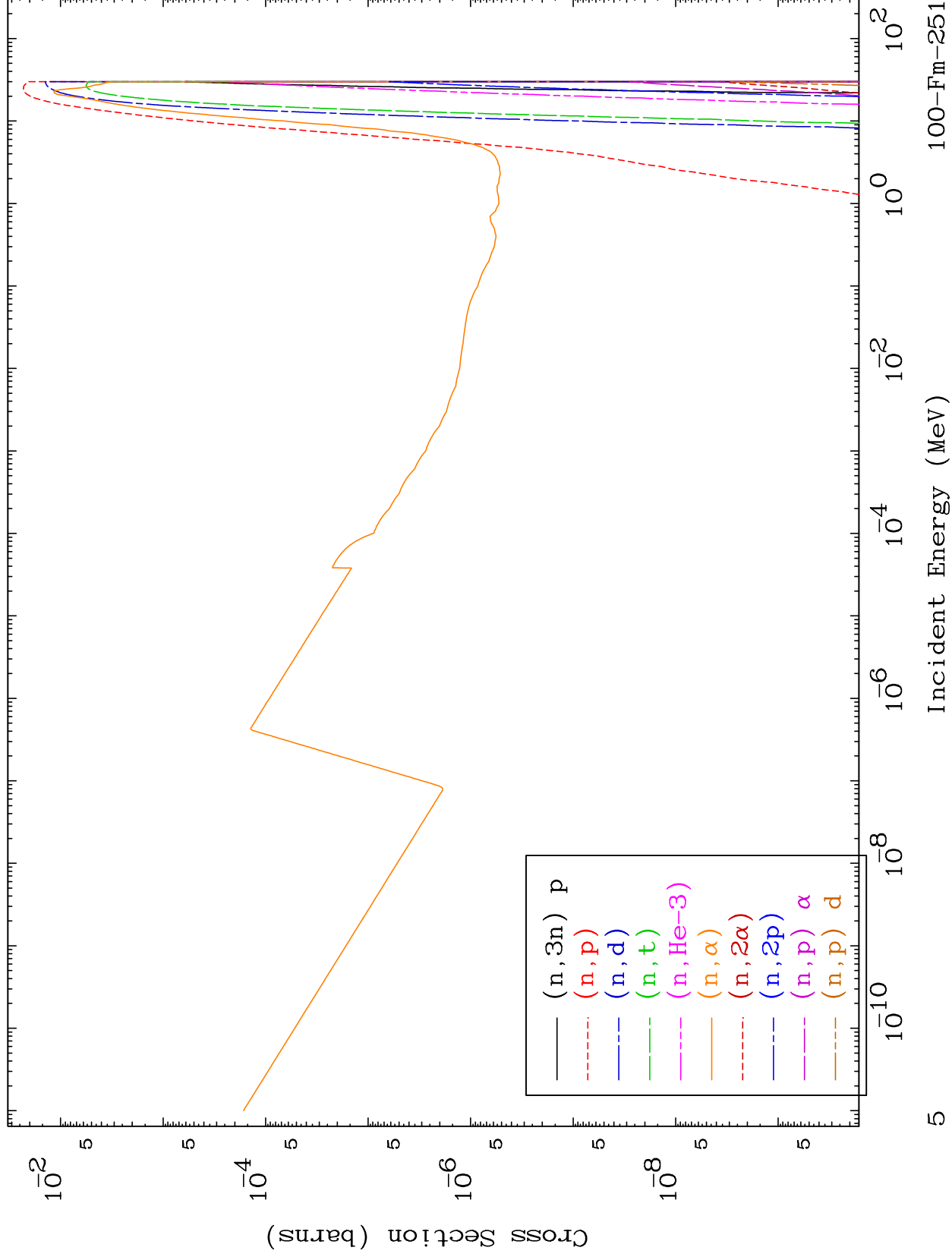


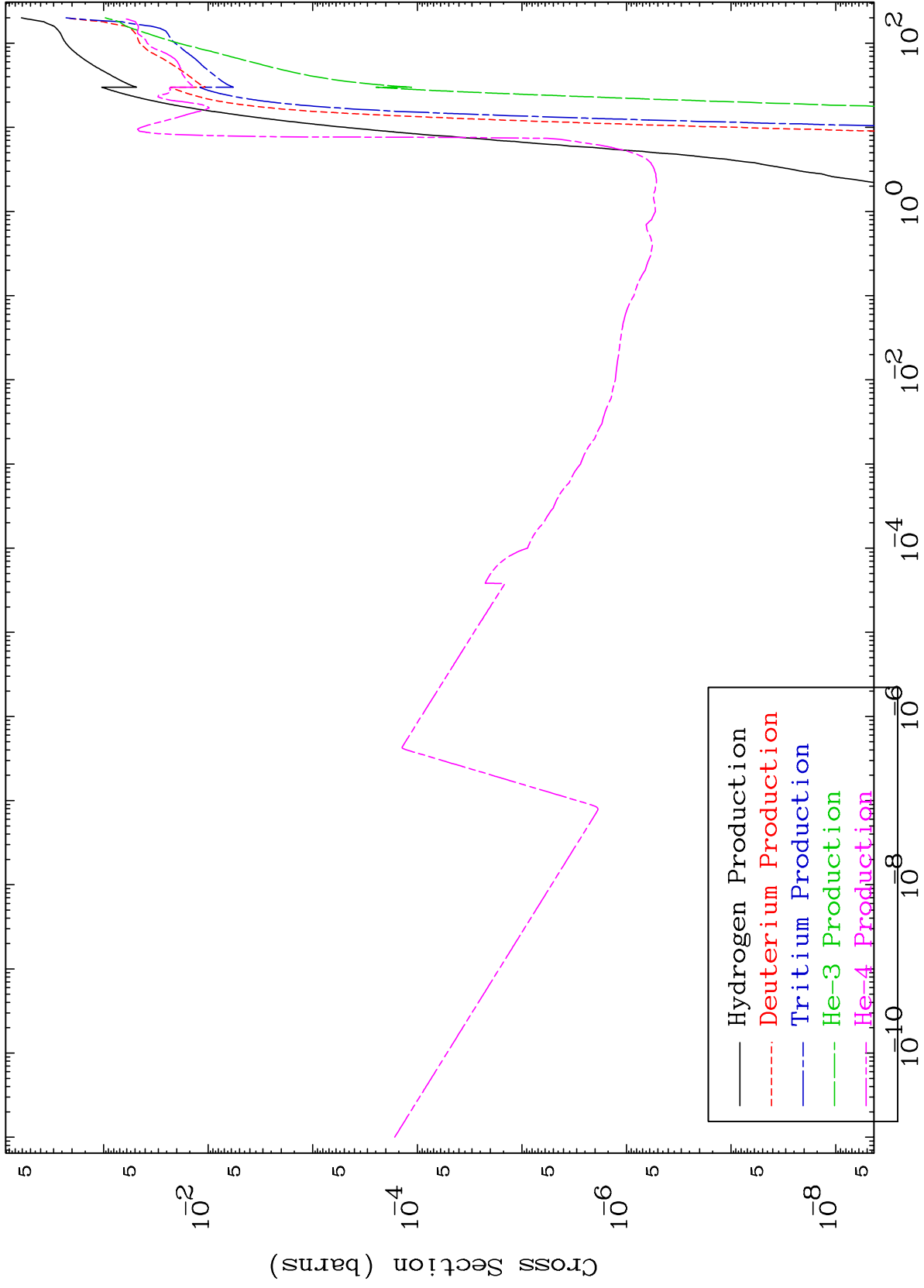


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Charged Particle
293 Kelvin Cross Sections

100-Fm-251



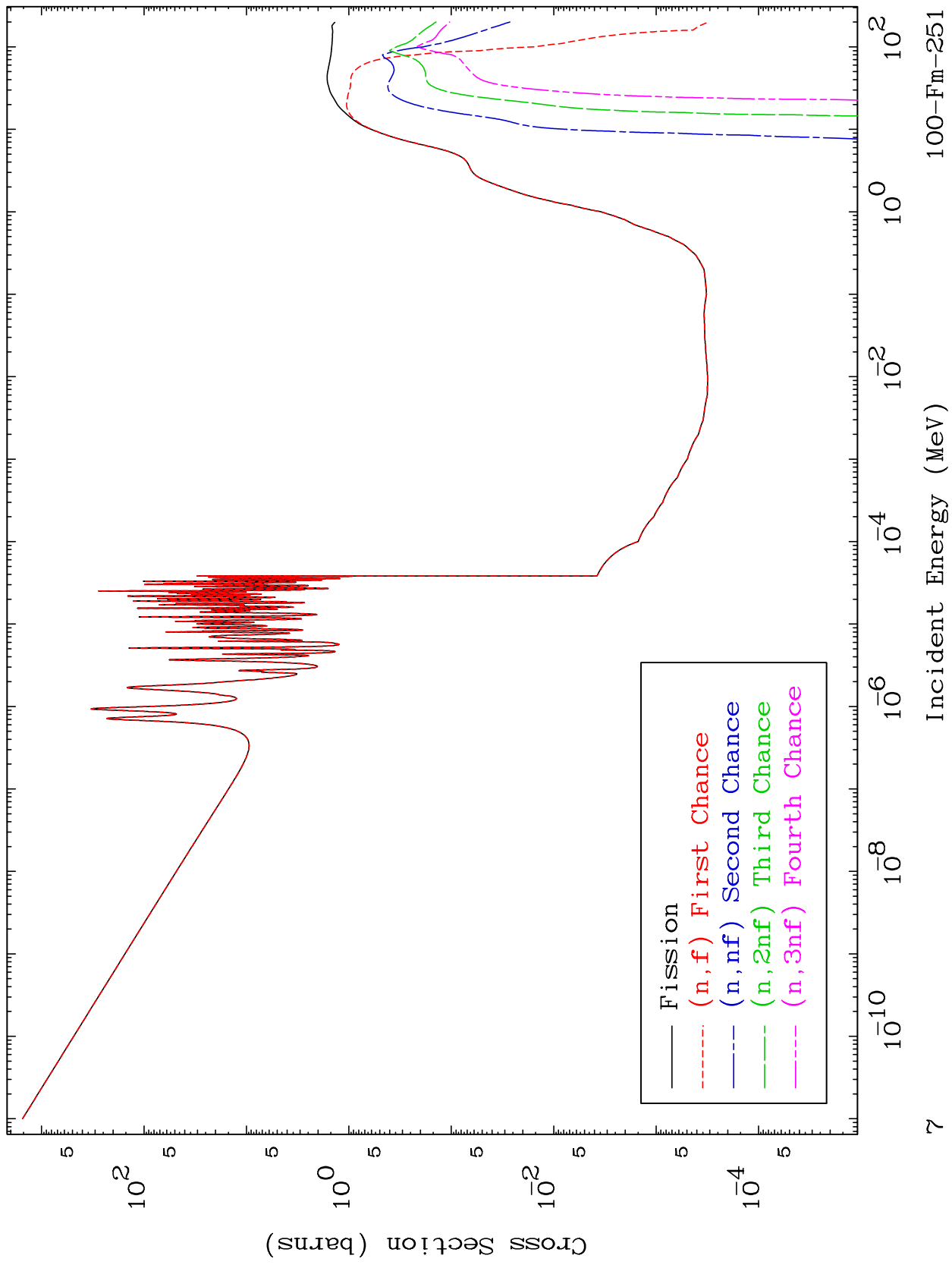


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Fission

100-Fm-251

293 Kelvin Cross Sections

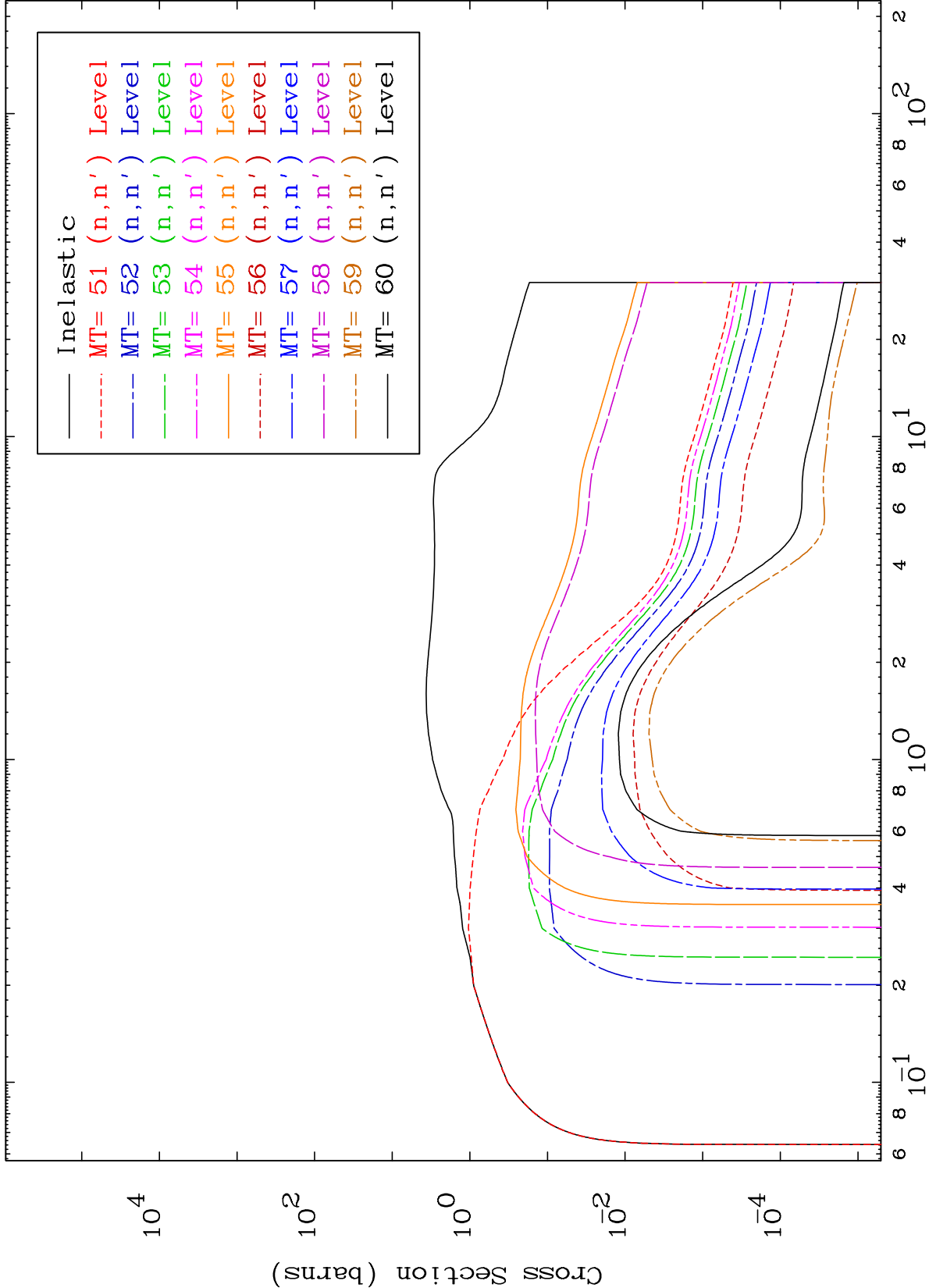


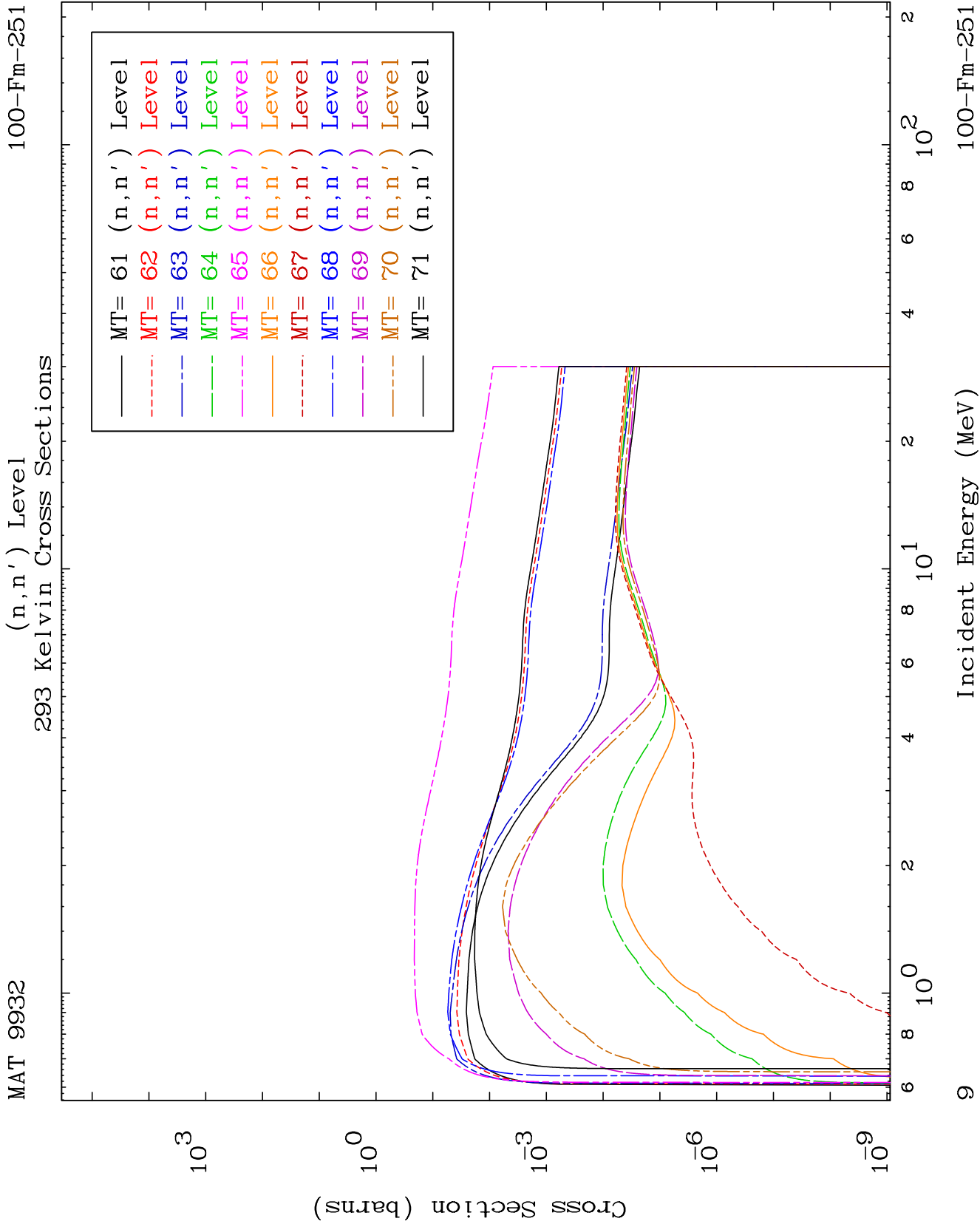
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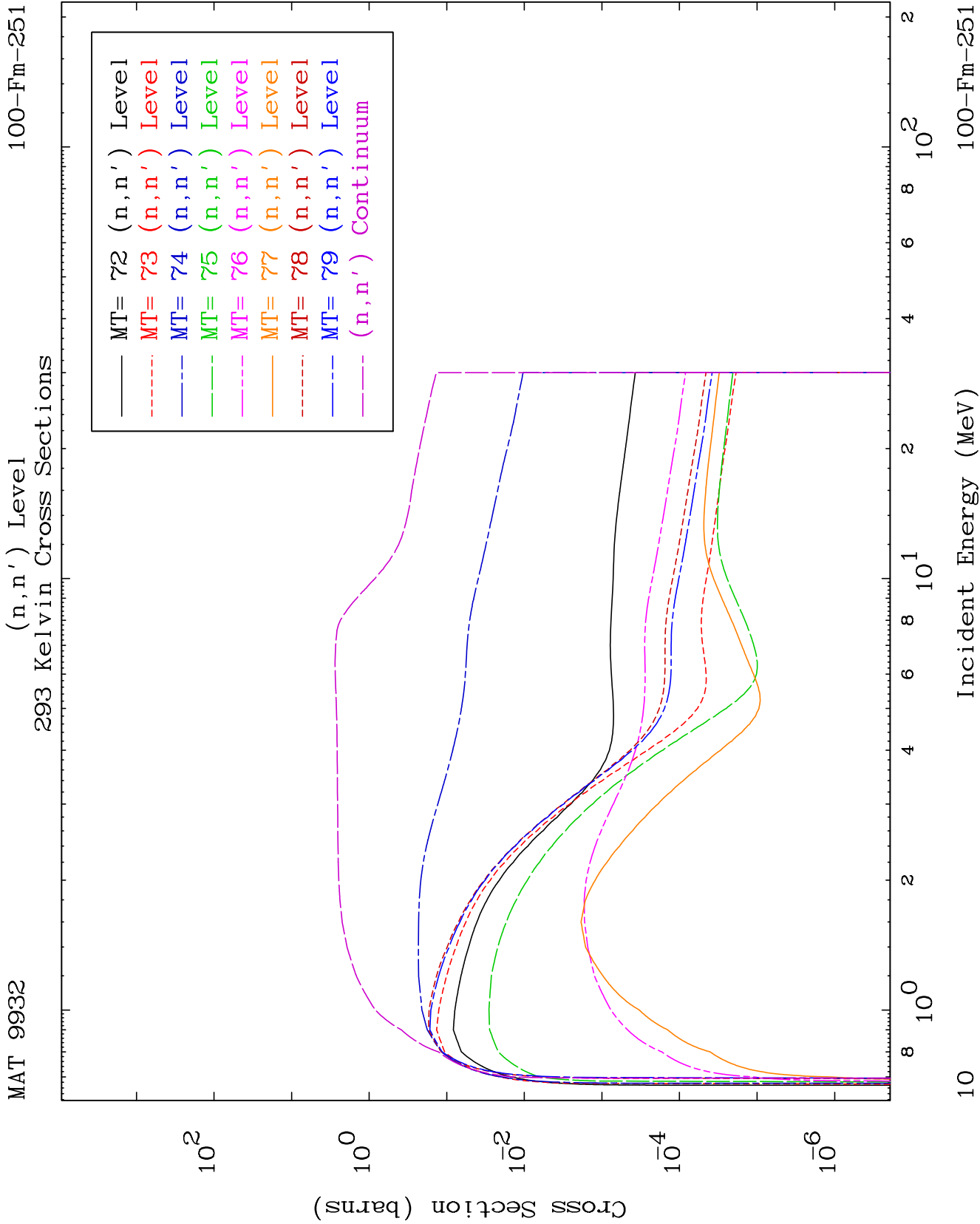
(n,n') Level

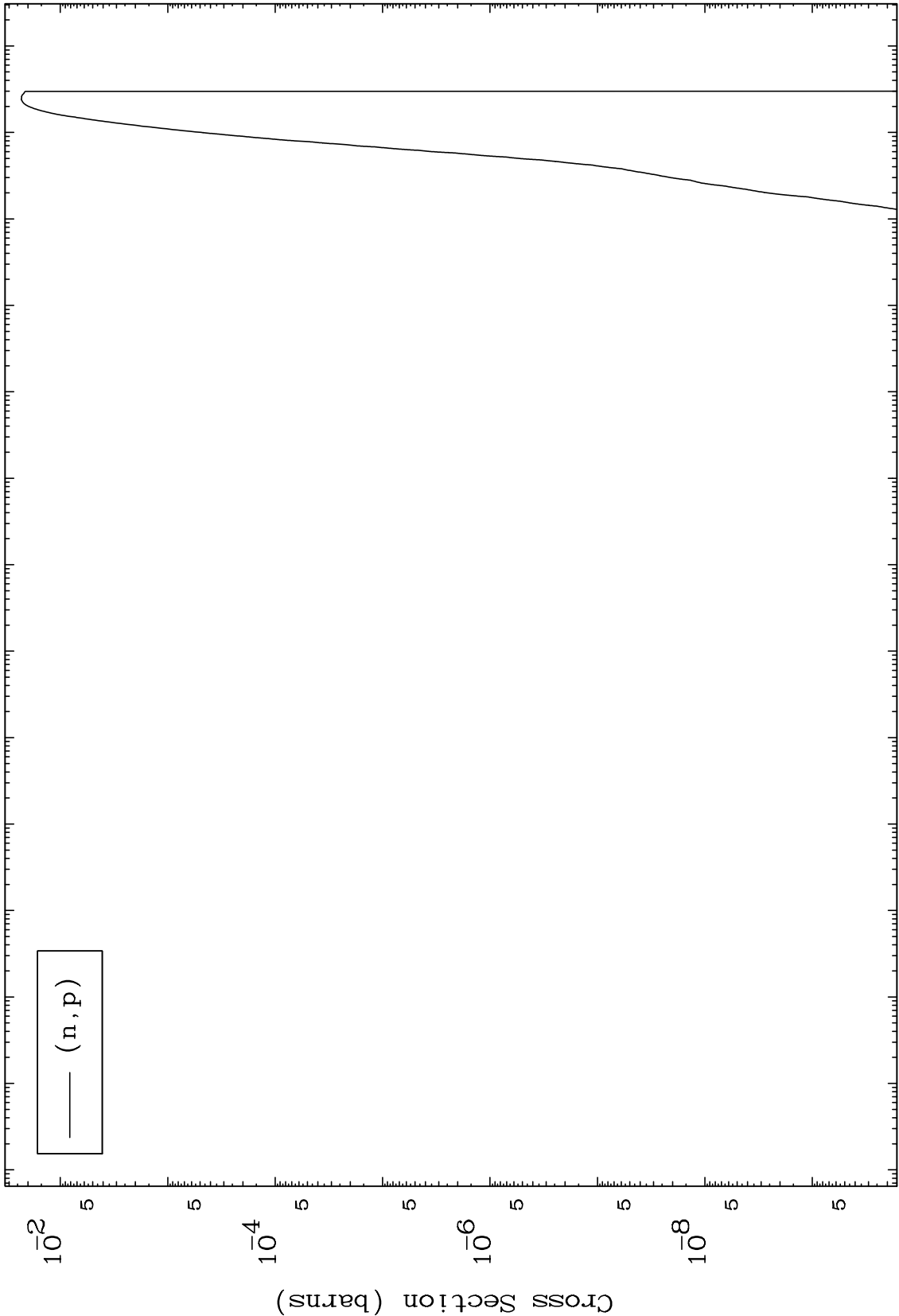
293 Kelvin Cross Sections

100-Fm-251





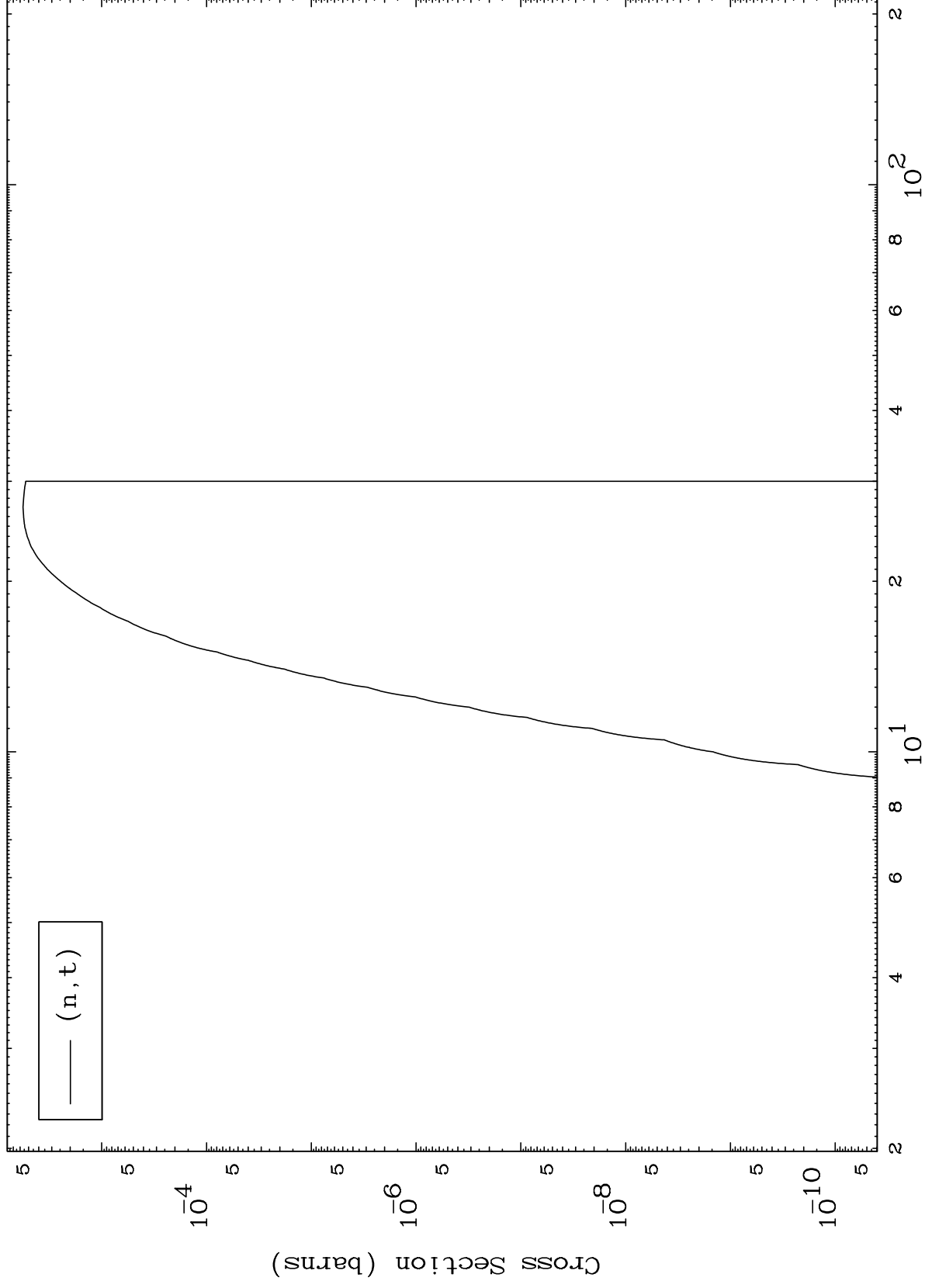




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(n,t) Levels
293 Kelvin Cross Sections

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Incident Energy (MeV)

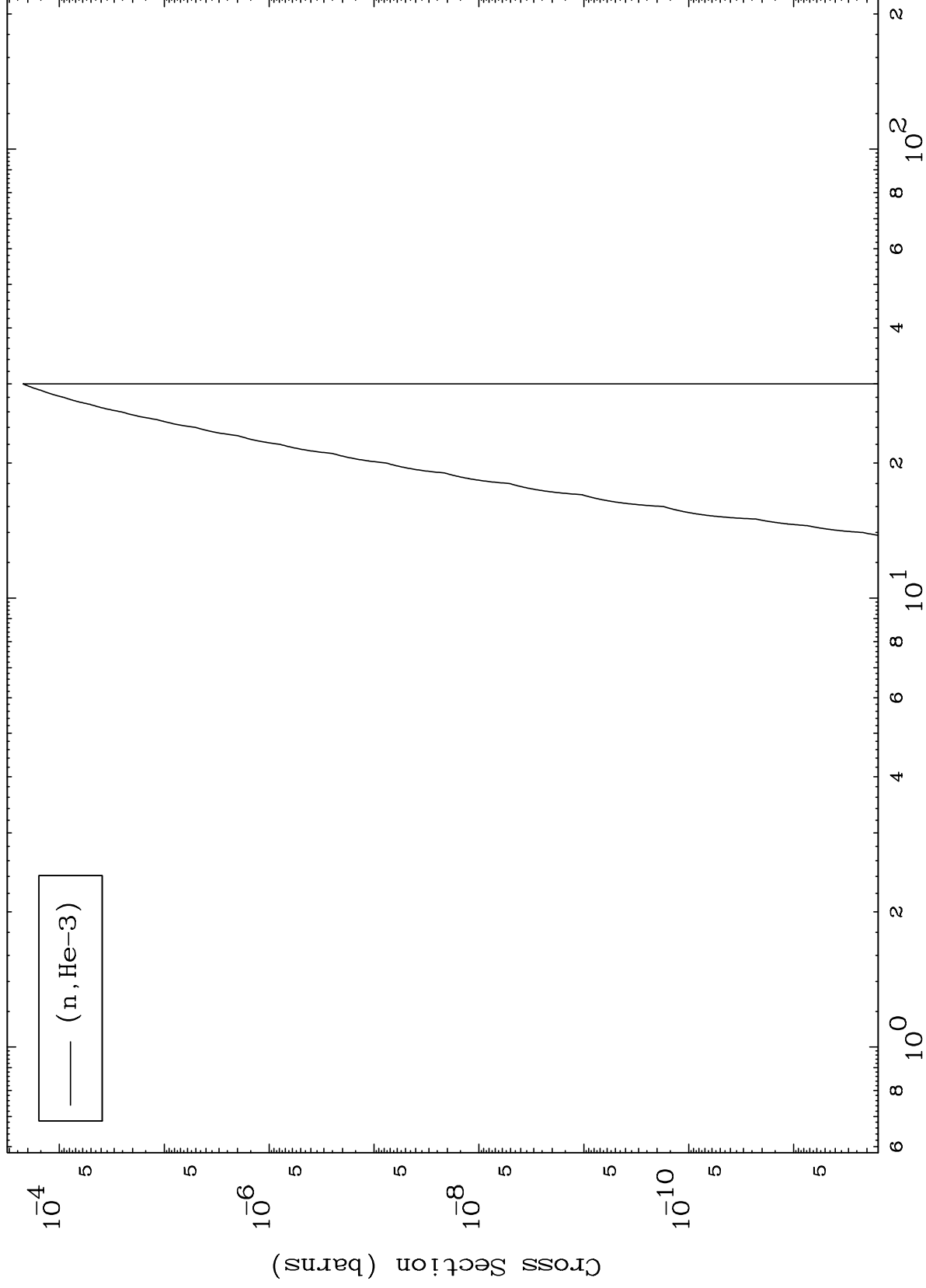
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(n,He3) Levels

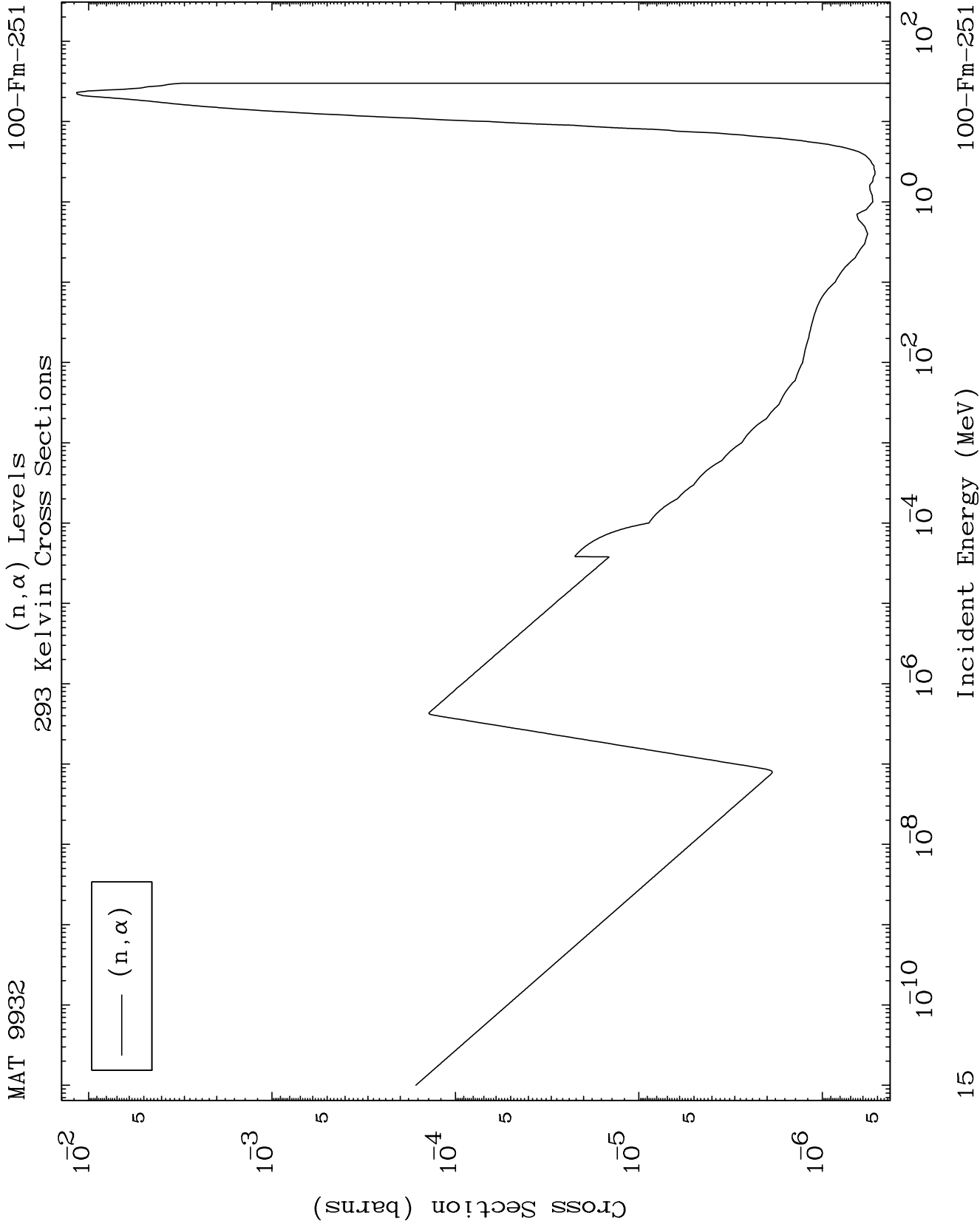
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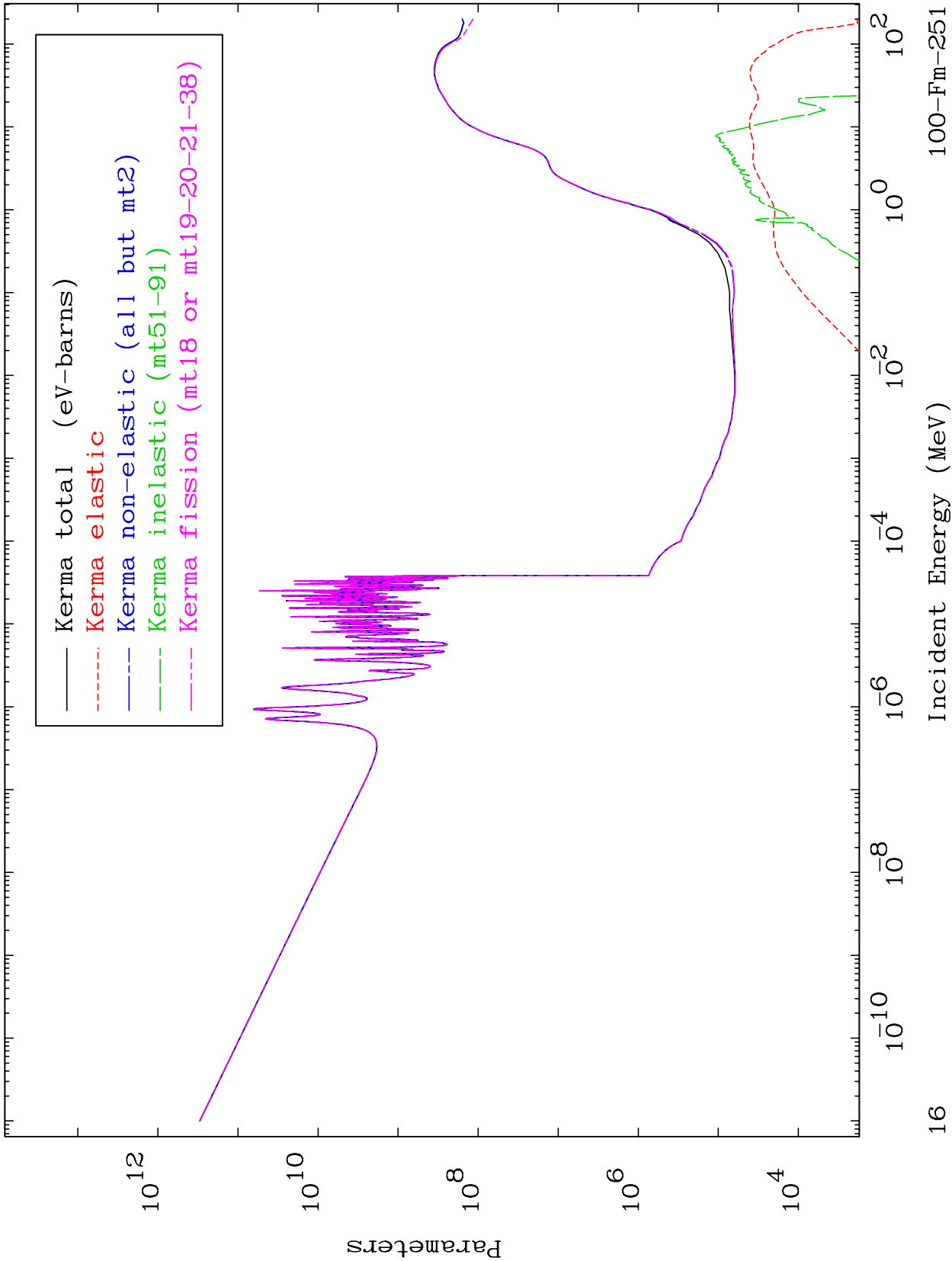
293 Kelvin Cross Sections

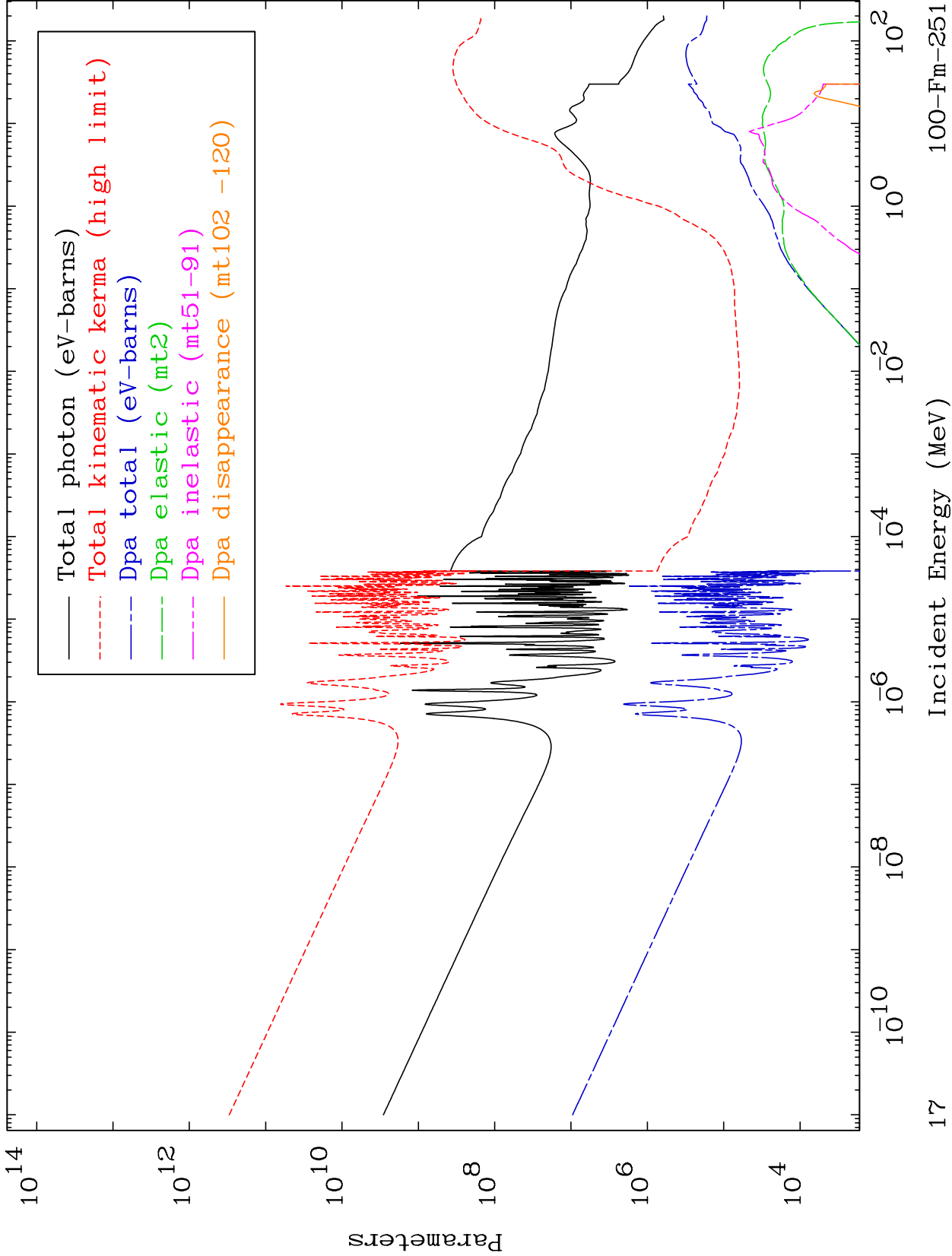


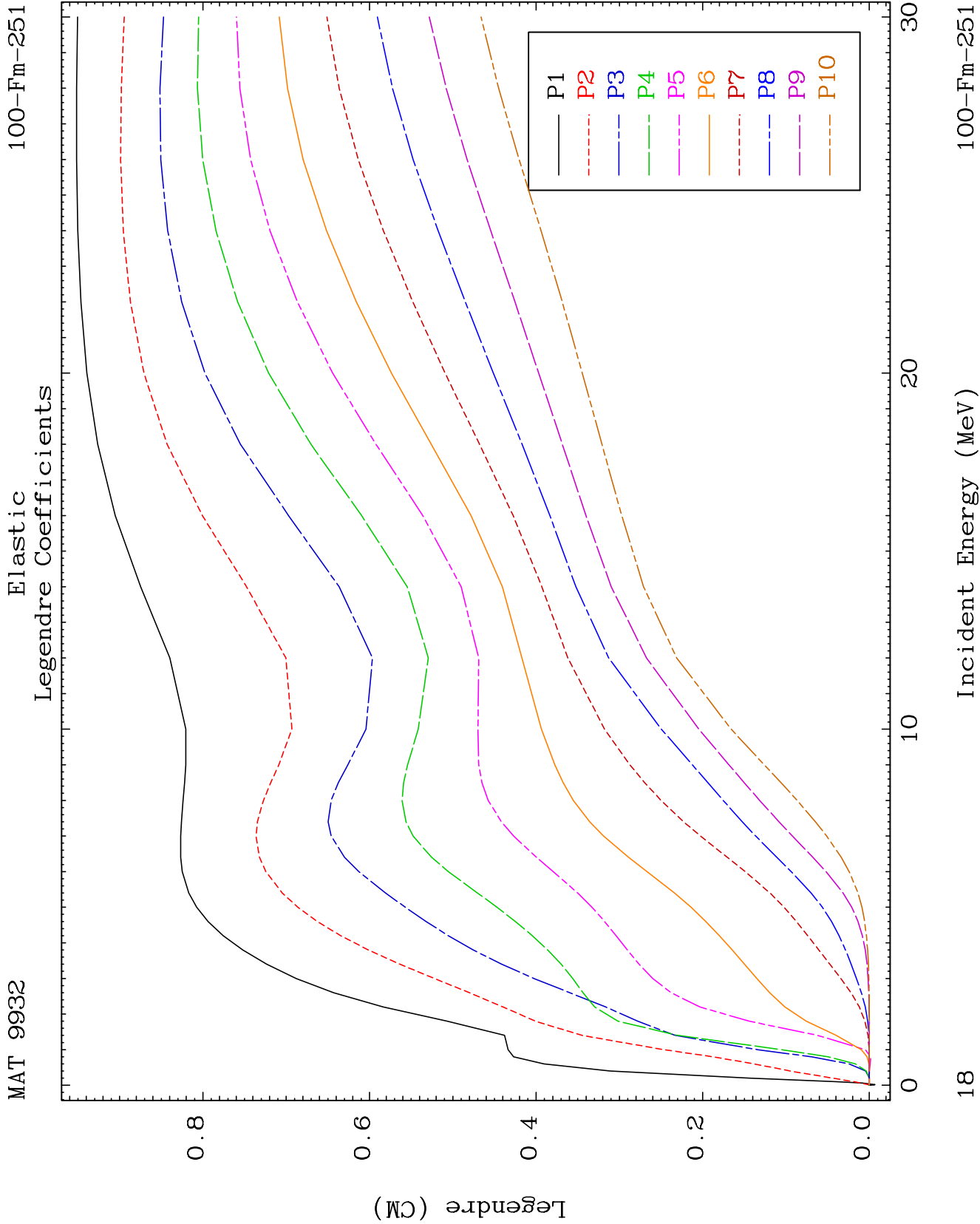
Incident Energy (MeV)

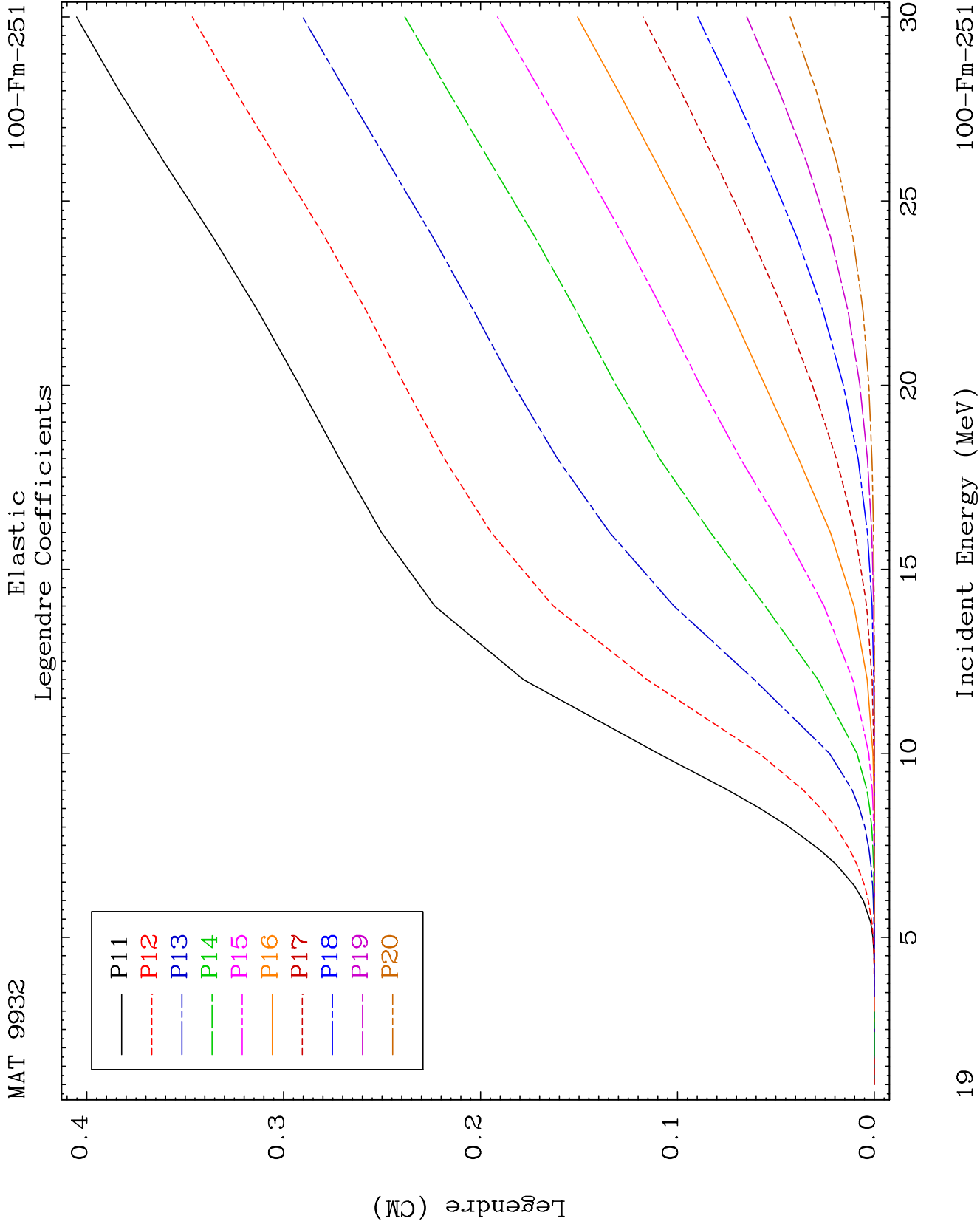
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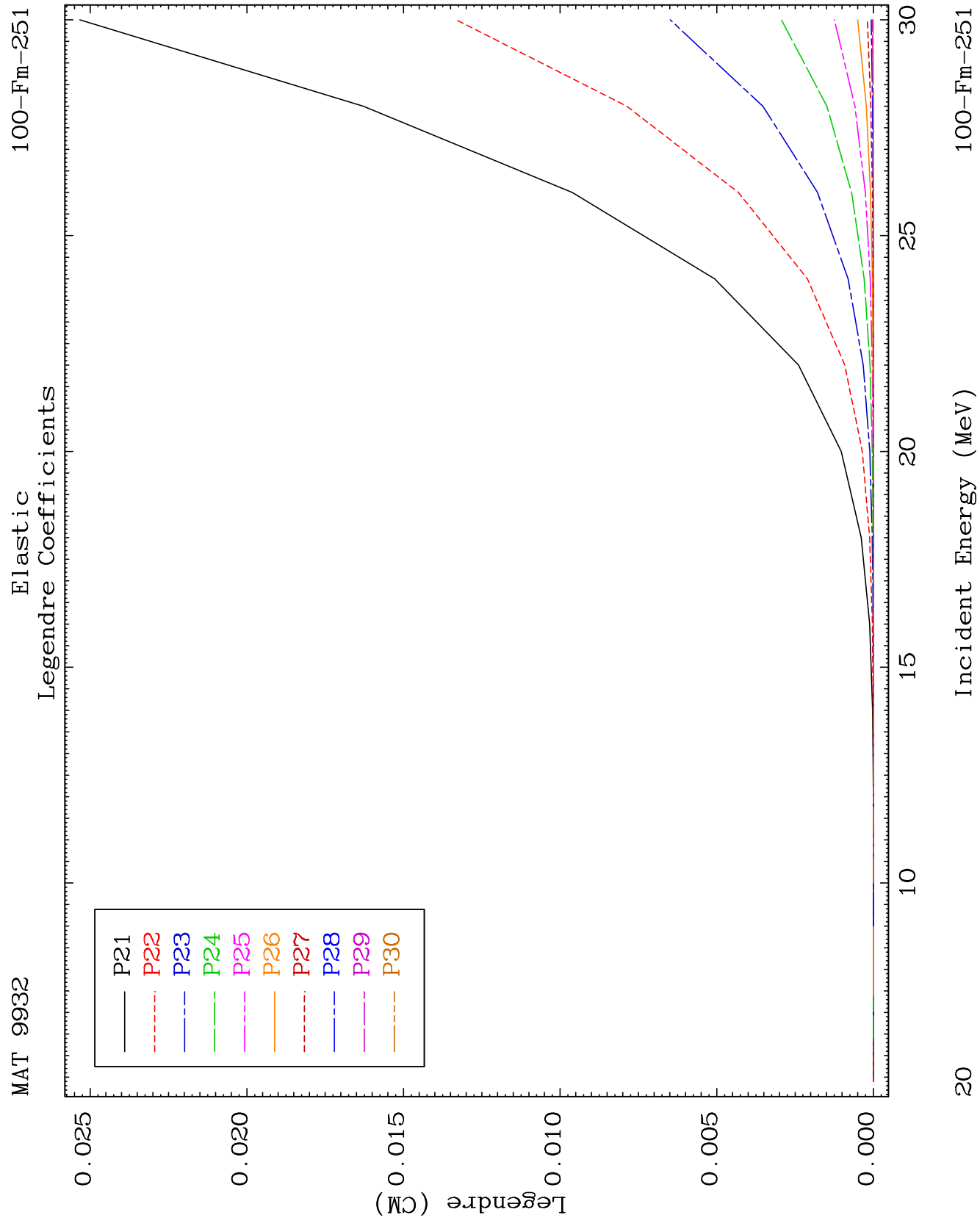


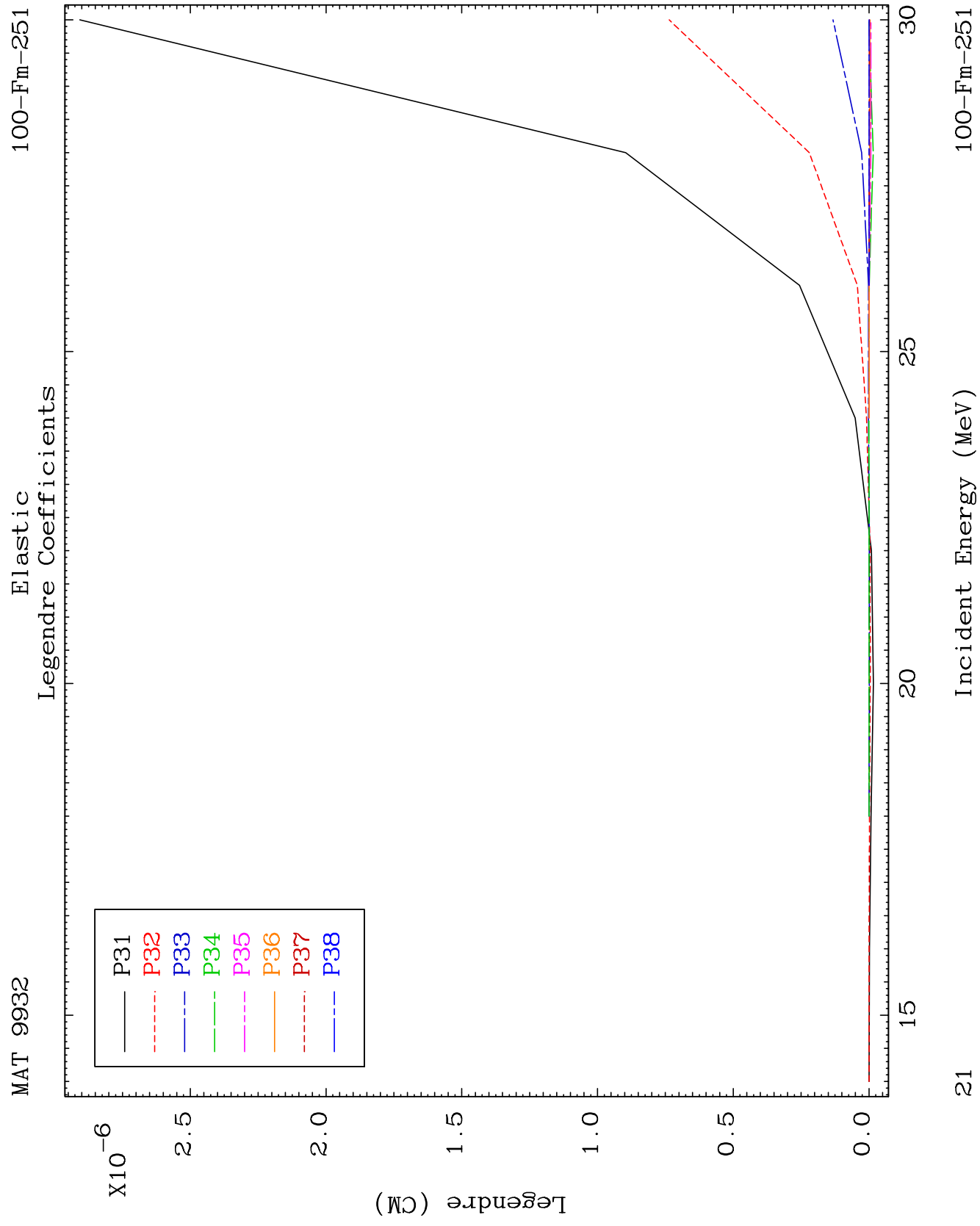








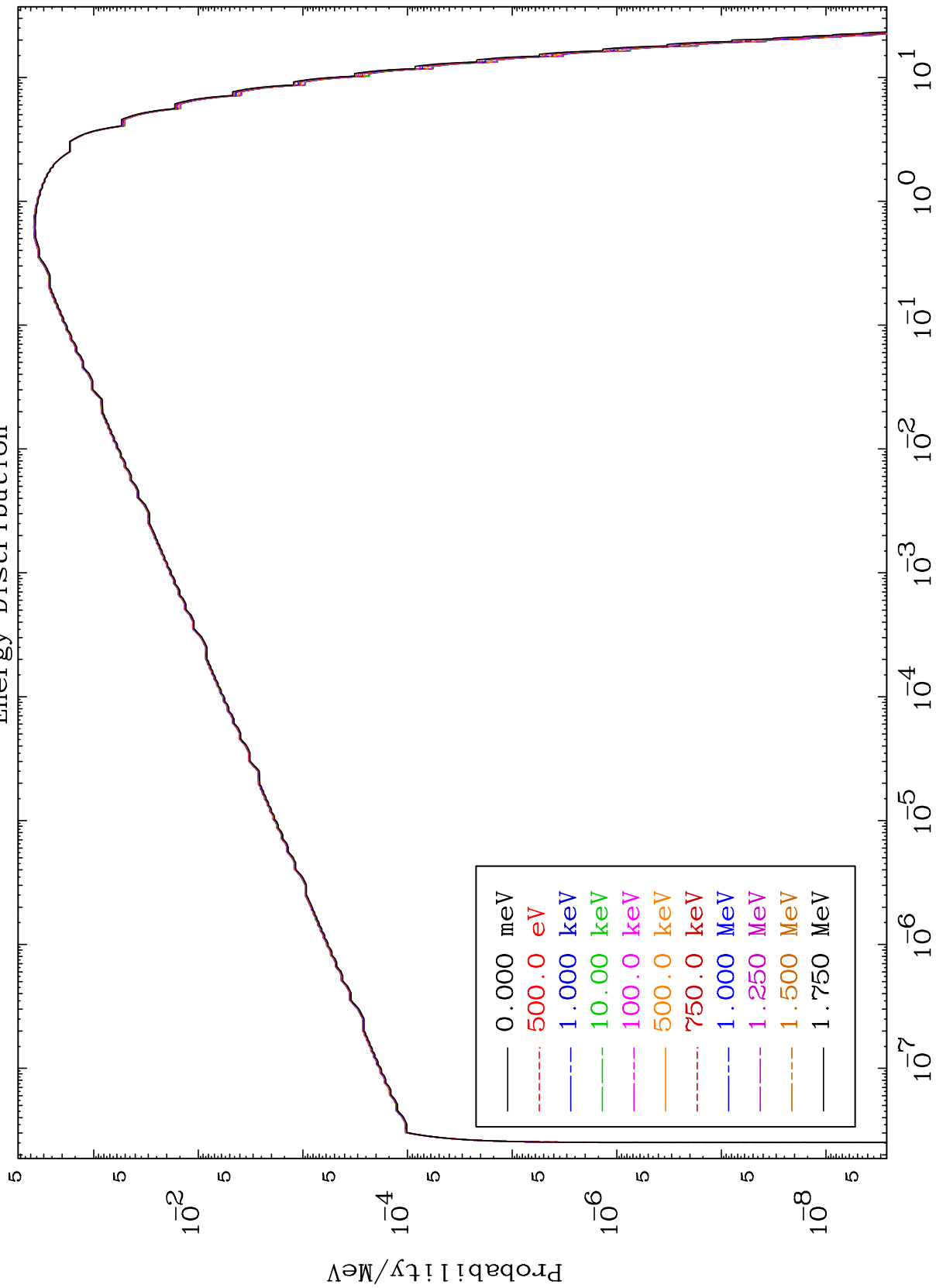




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Fission
Energy Distribution

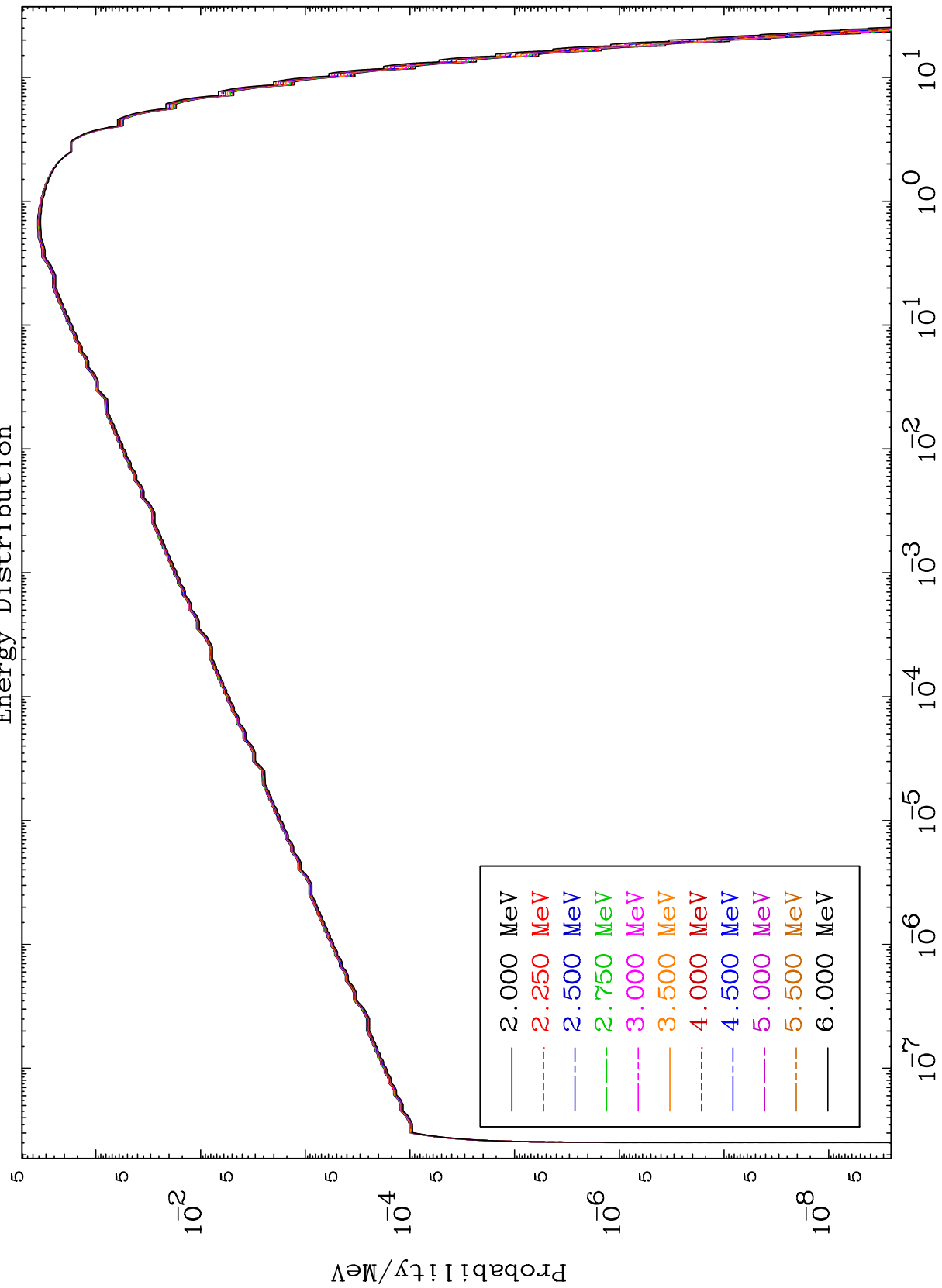
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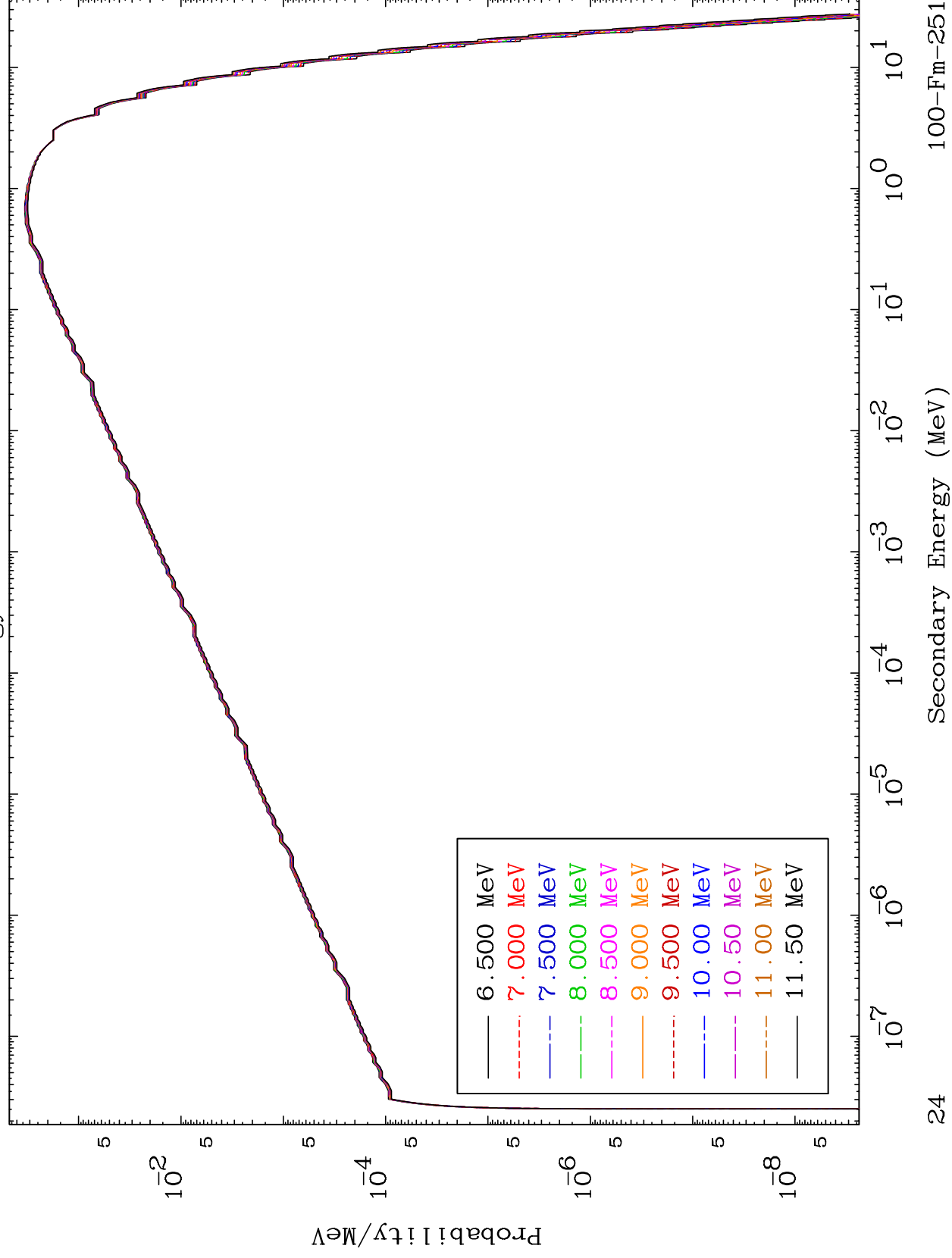


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Fission
Energy Distribution

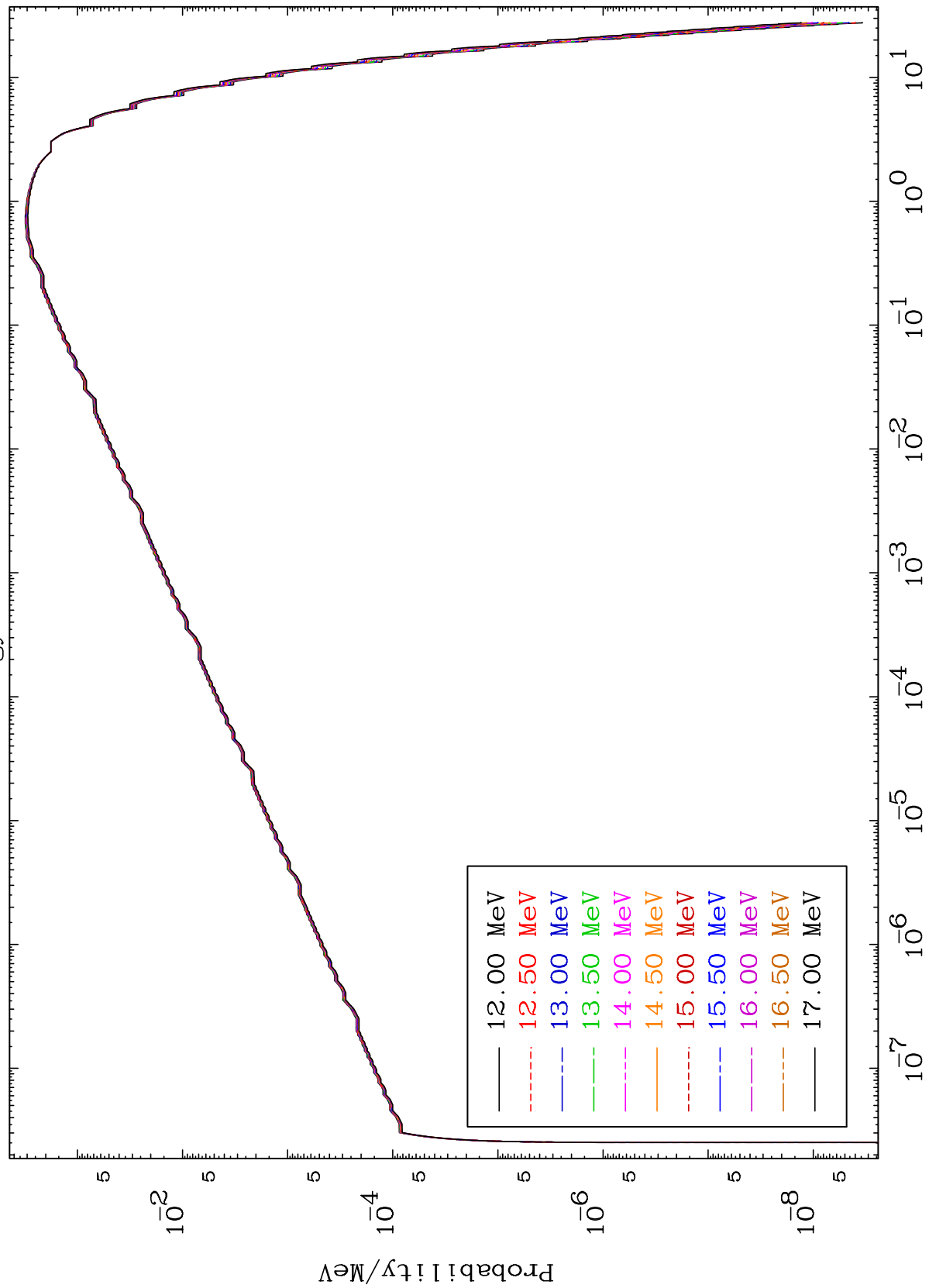
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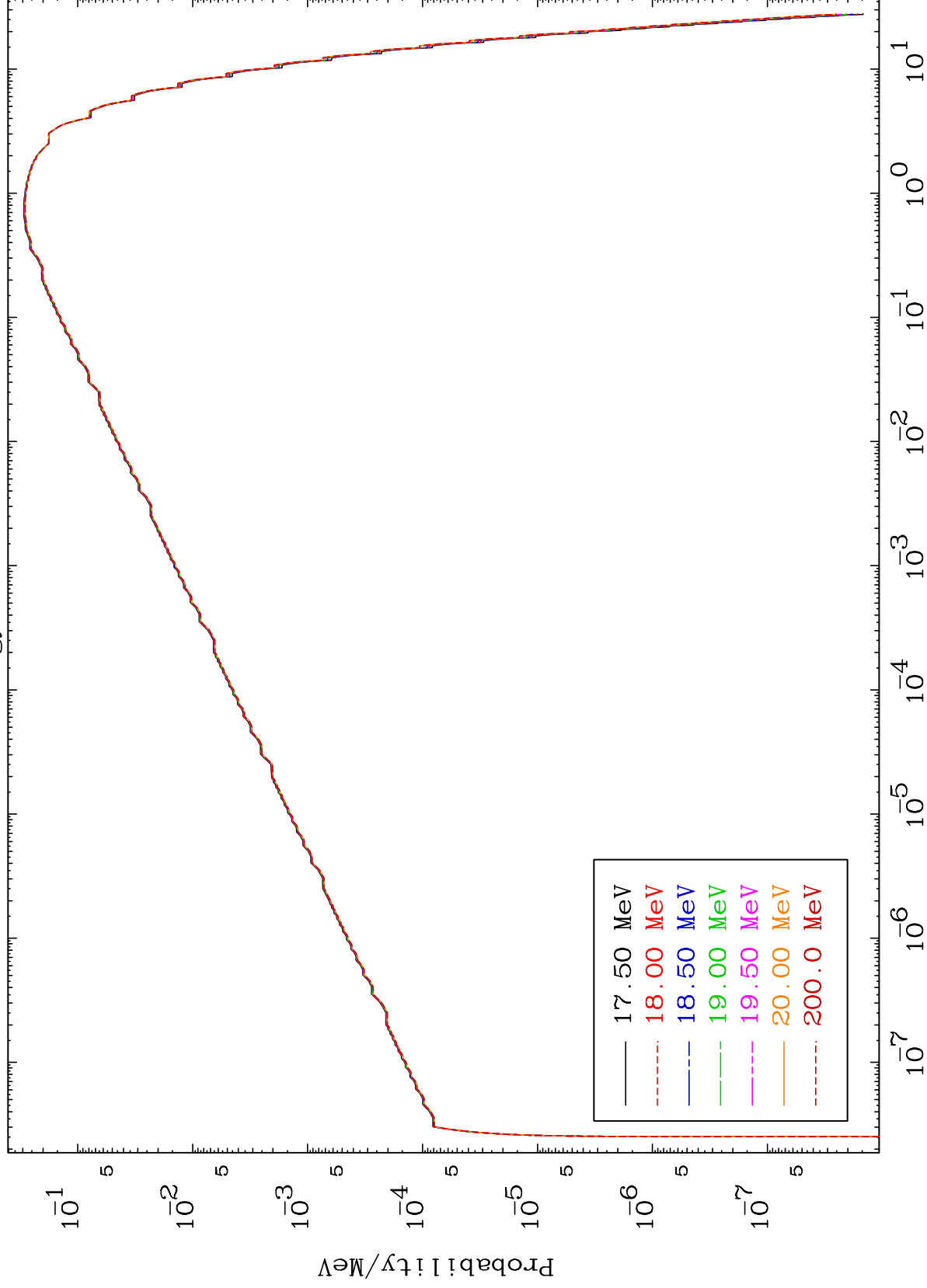
Fission
Energy Distribution

Fission Energy Distribution

100-Fm-251



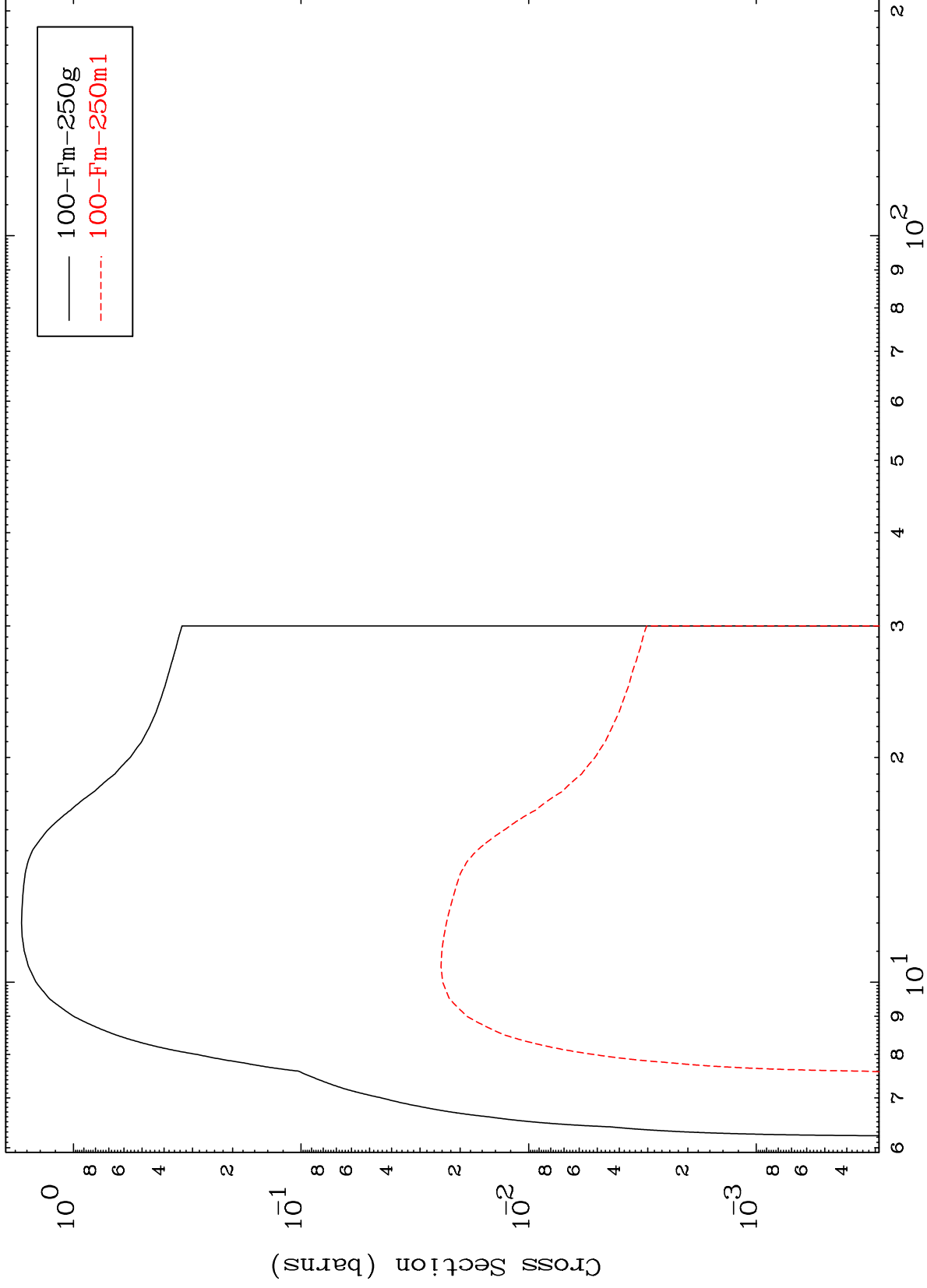
Fission Energy Distribution



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(n,2n)
Radionuclide Production Cross Section



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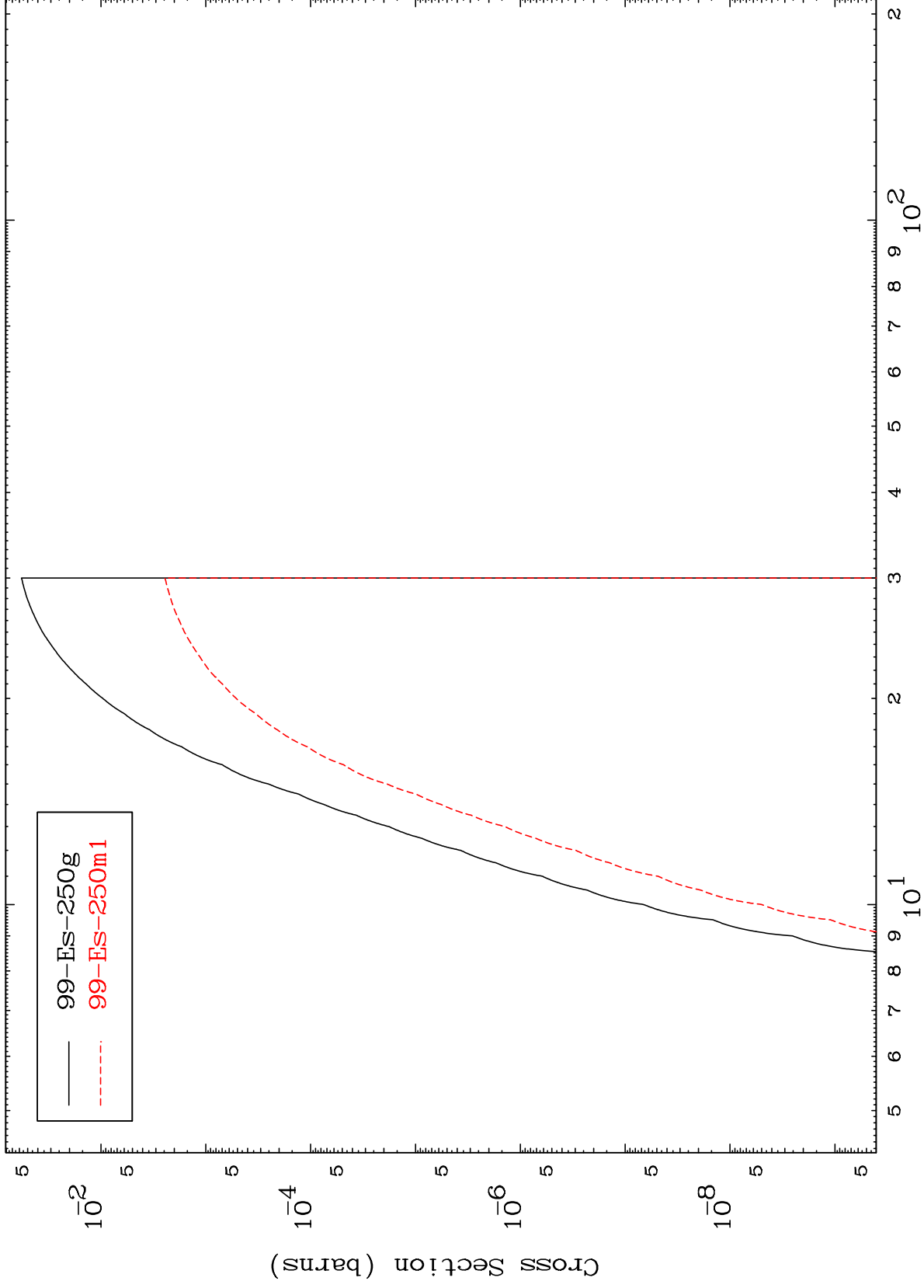
Incident Energy (MeV)

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(n,n') p
Radionuclide Production Cross Section



28

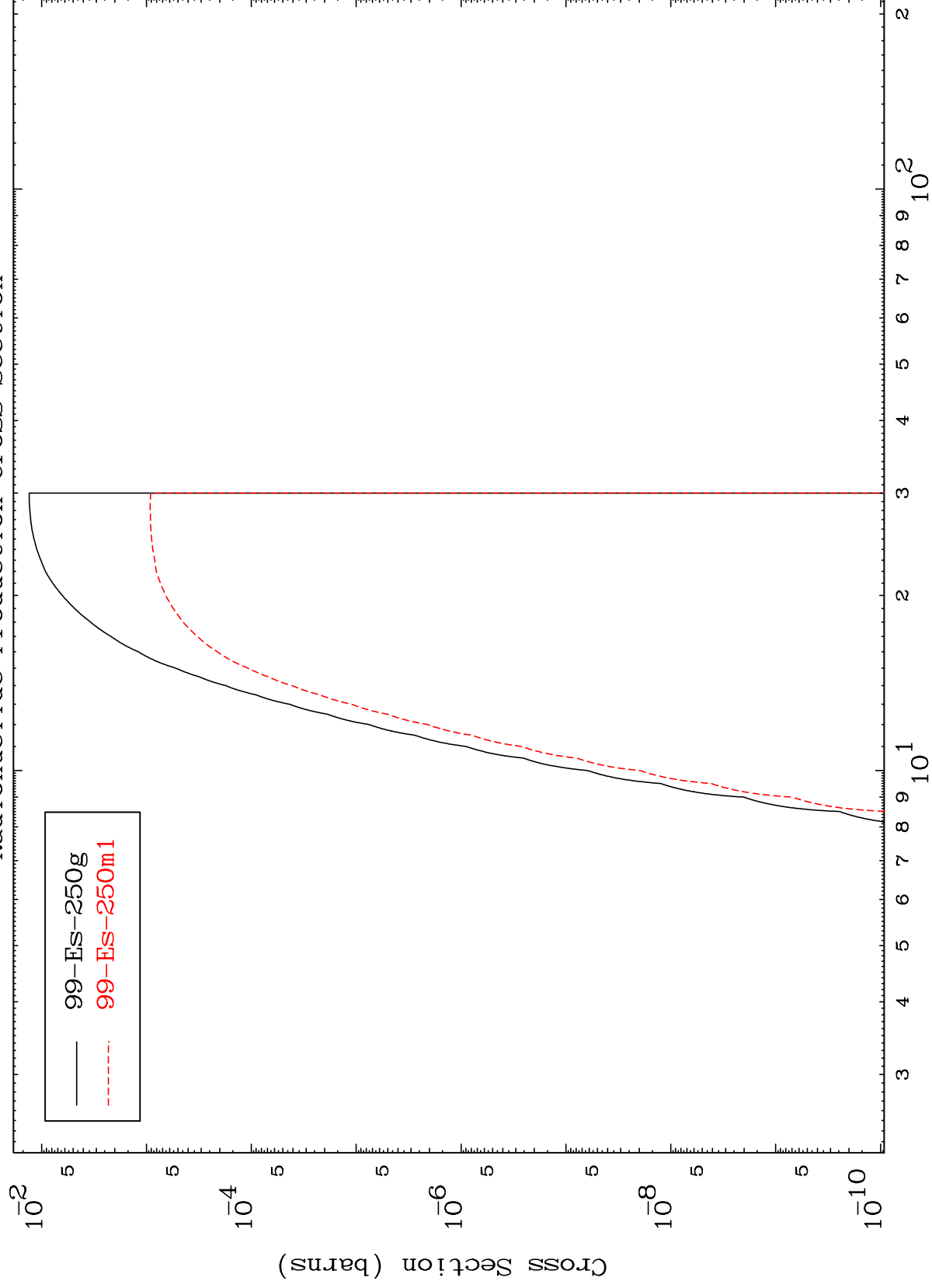
Incident Energy (MeV)

100-Fm-251

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100-Fm-251

Radionuclide Production Cross Section
(n,d)



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Incident Energy (MeV)

100-Fm-251