

Program EVALPLOT
(Version 2017-1)

by

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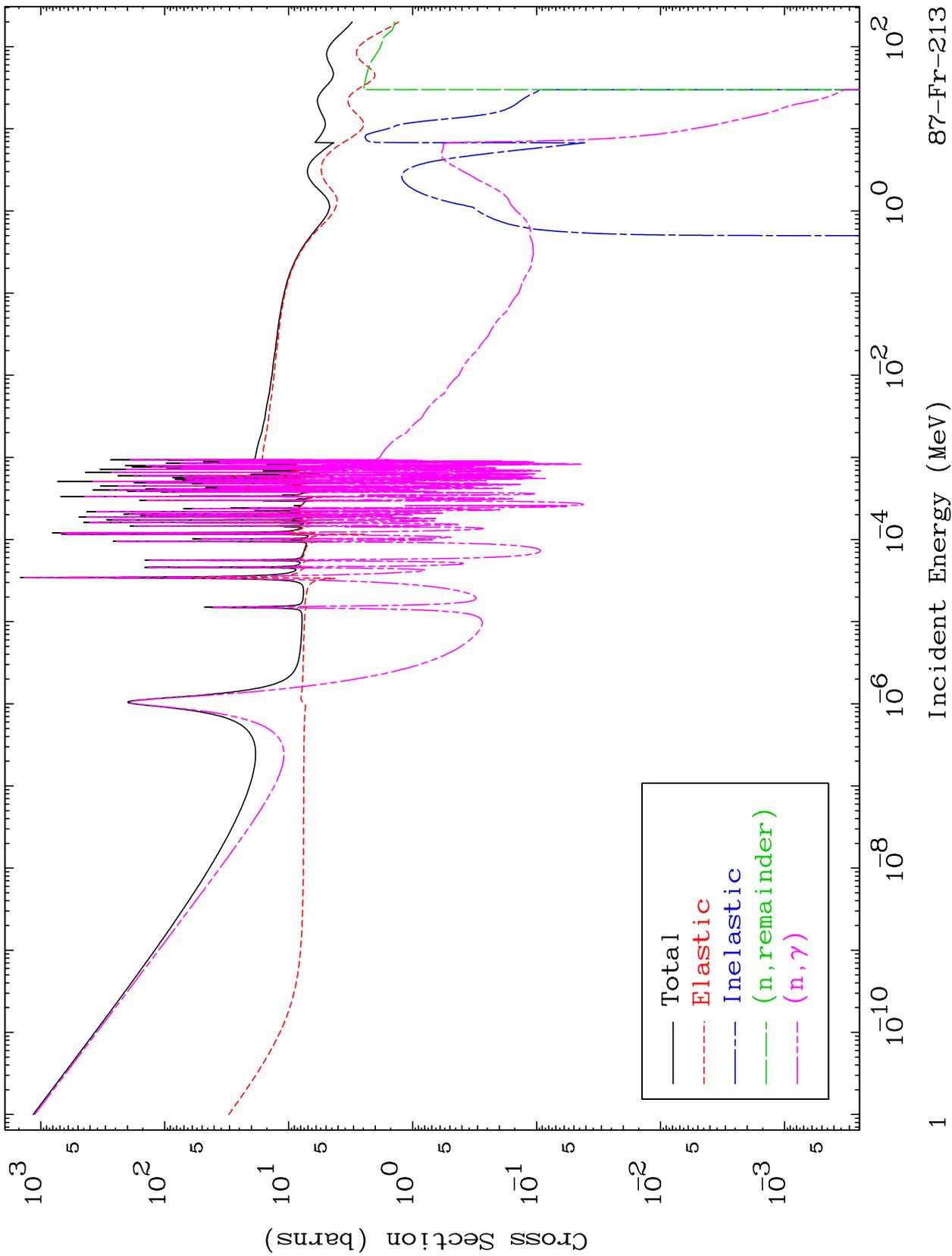
Press Mouse Button to Start

MAT 8728

Major

293 Kelvin Cross Sections

87-Fr-213



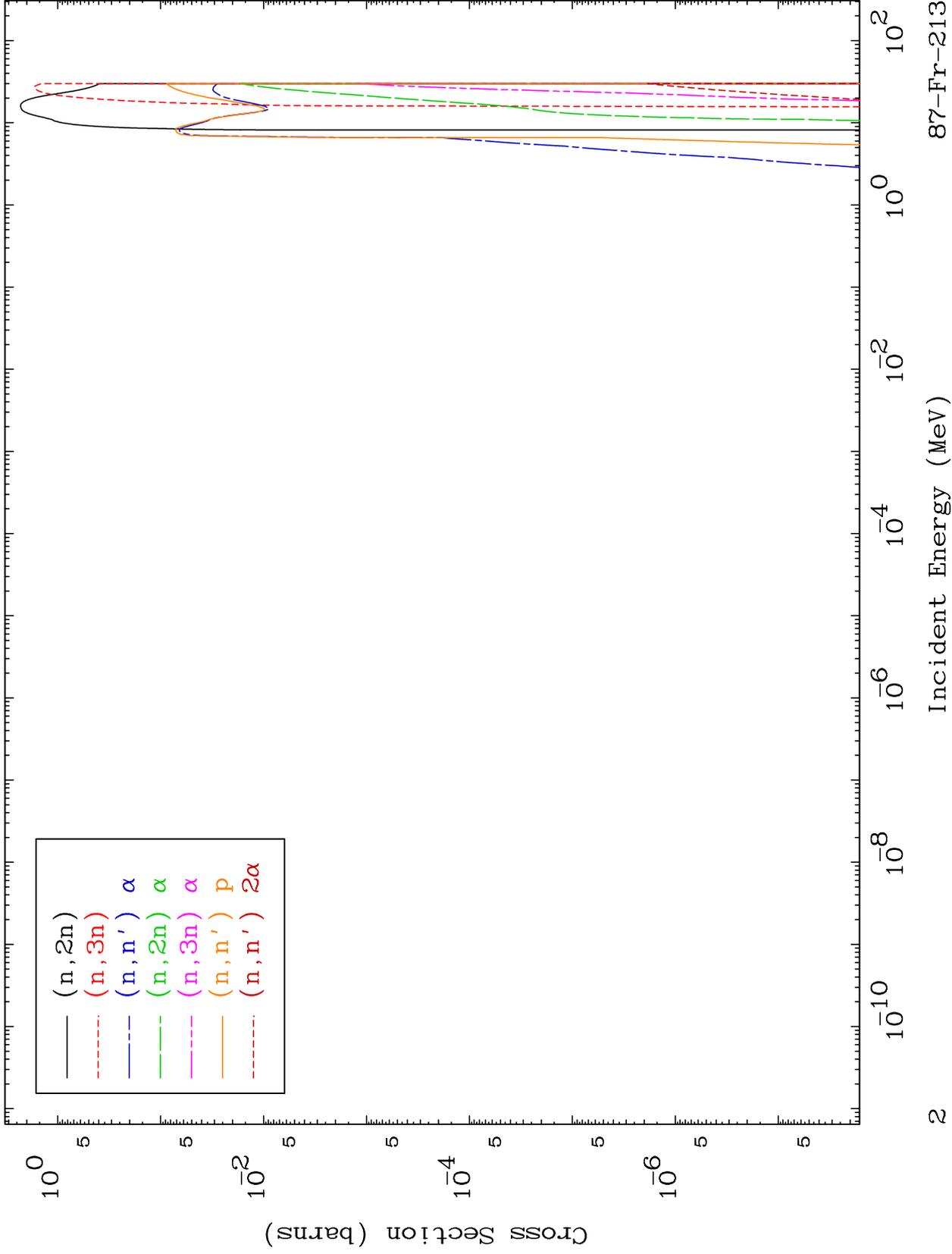
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Incident Energy (MeV)

MAT 8728

Neutron Production
293 Kelvin Cross Sections

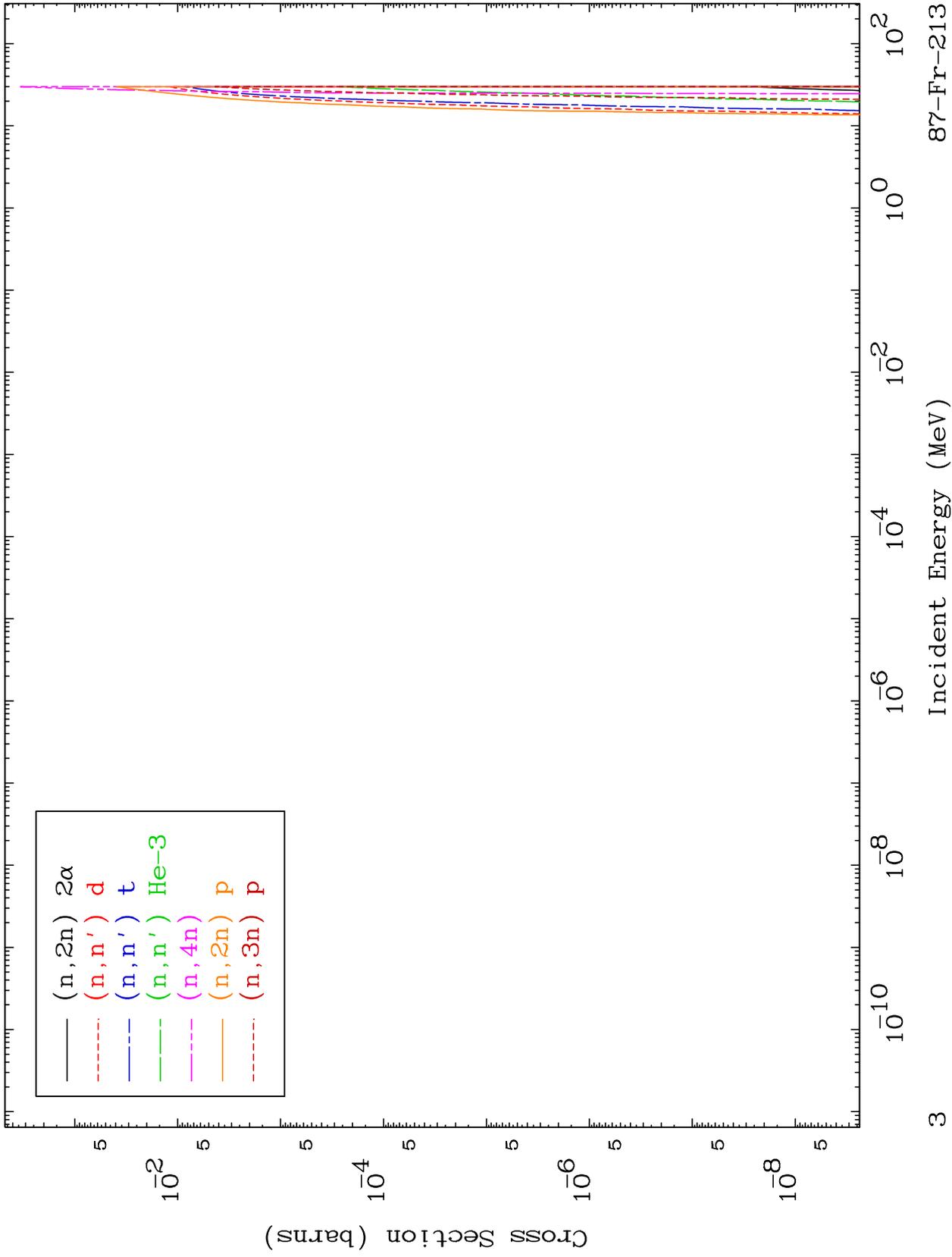
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MAT 8728

Neutron Production
293 Kelvin Cross Sections

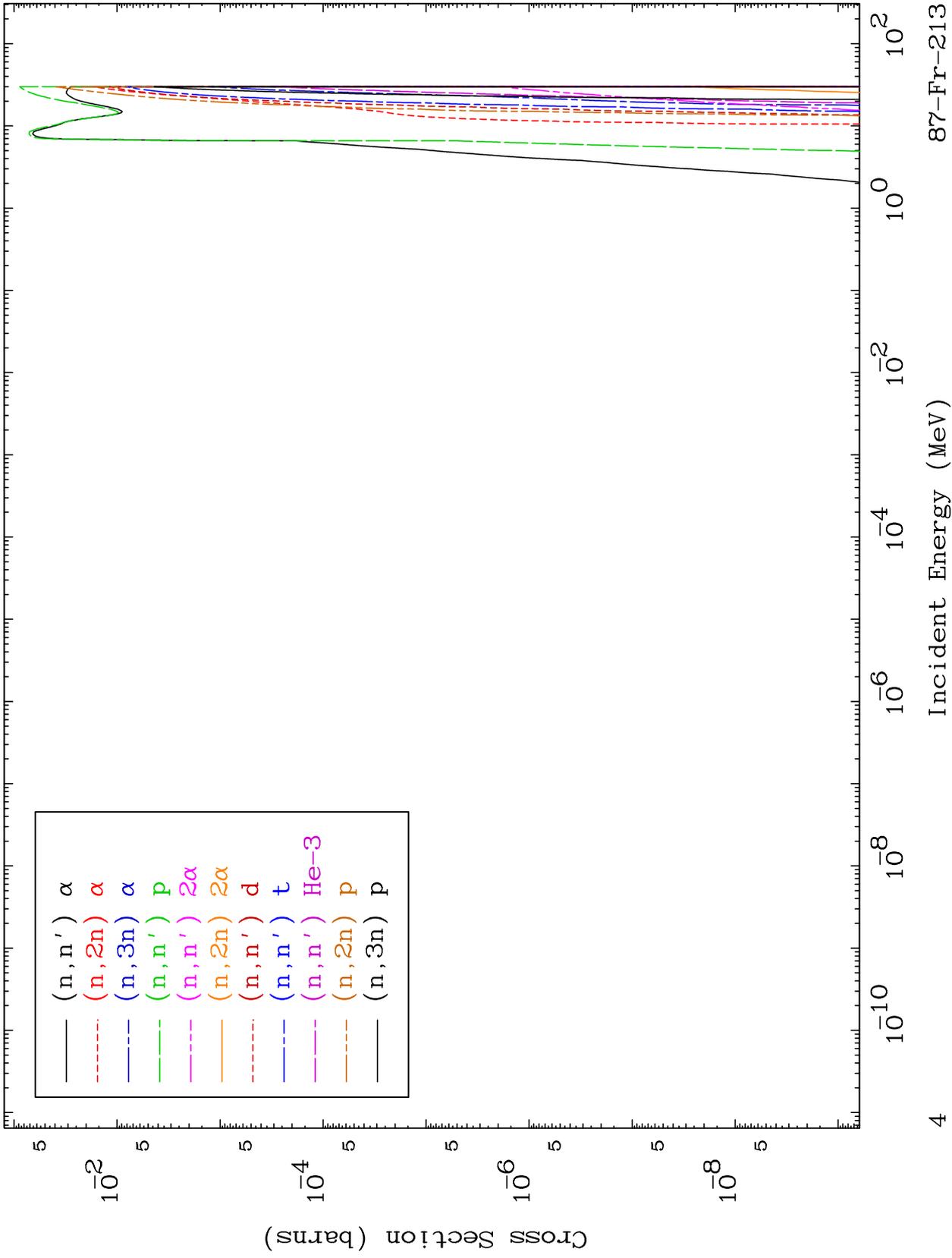
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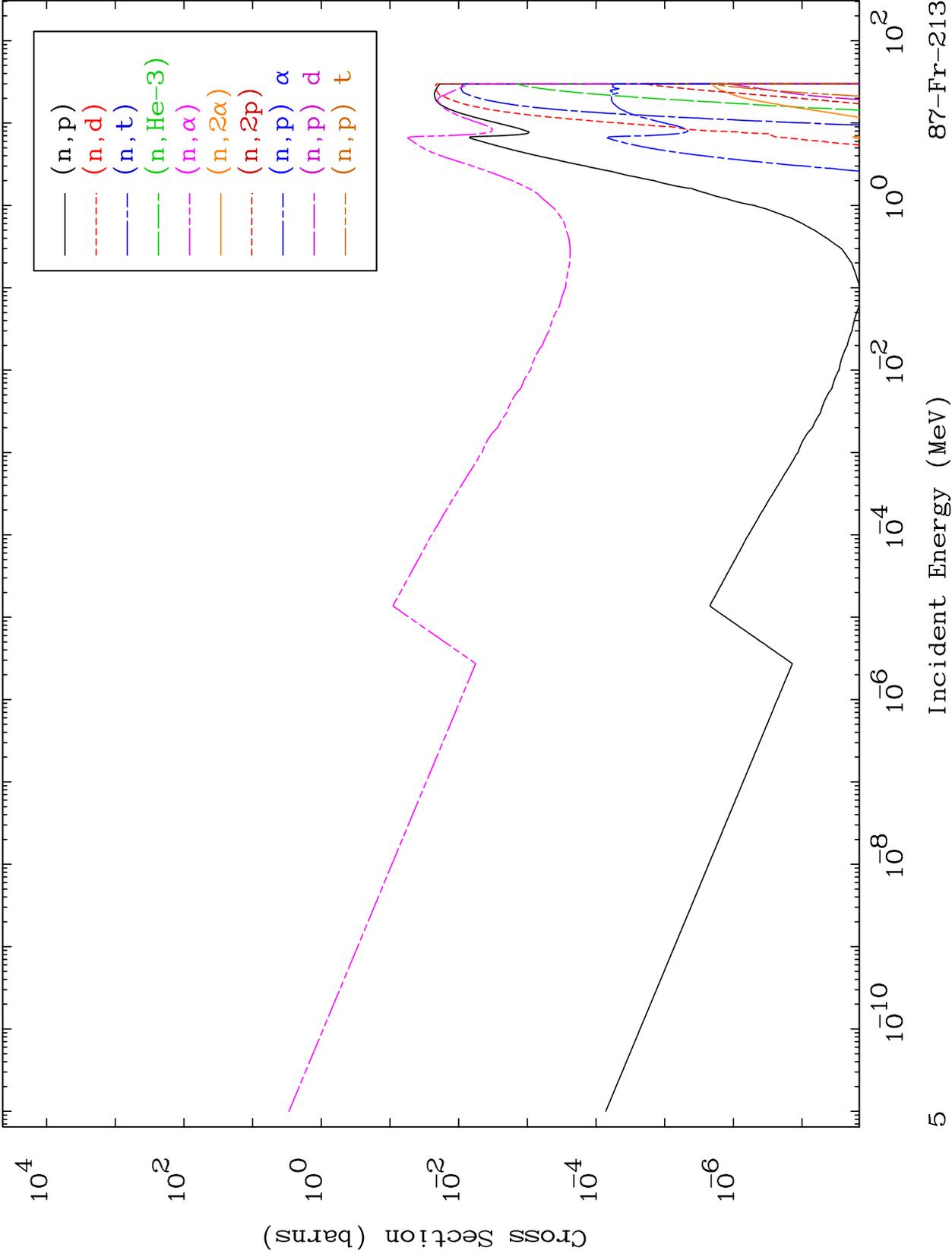


MAT 8728

Charged Particle
293 Kelvin Cross Sections

87-Fr-213

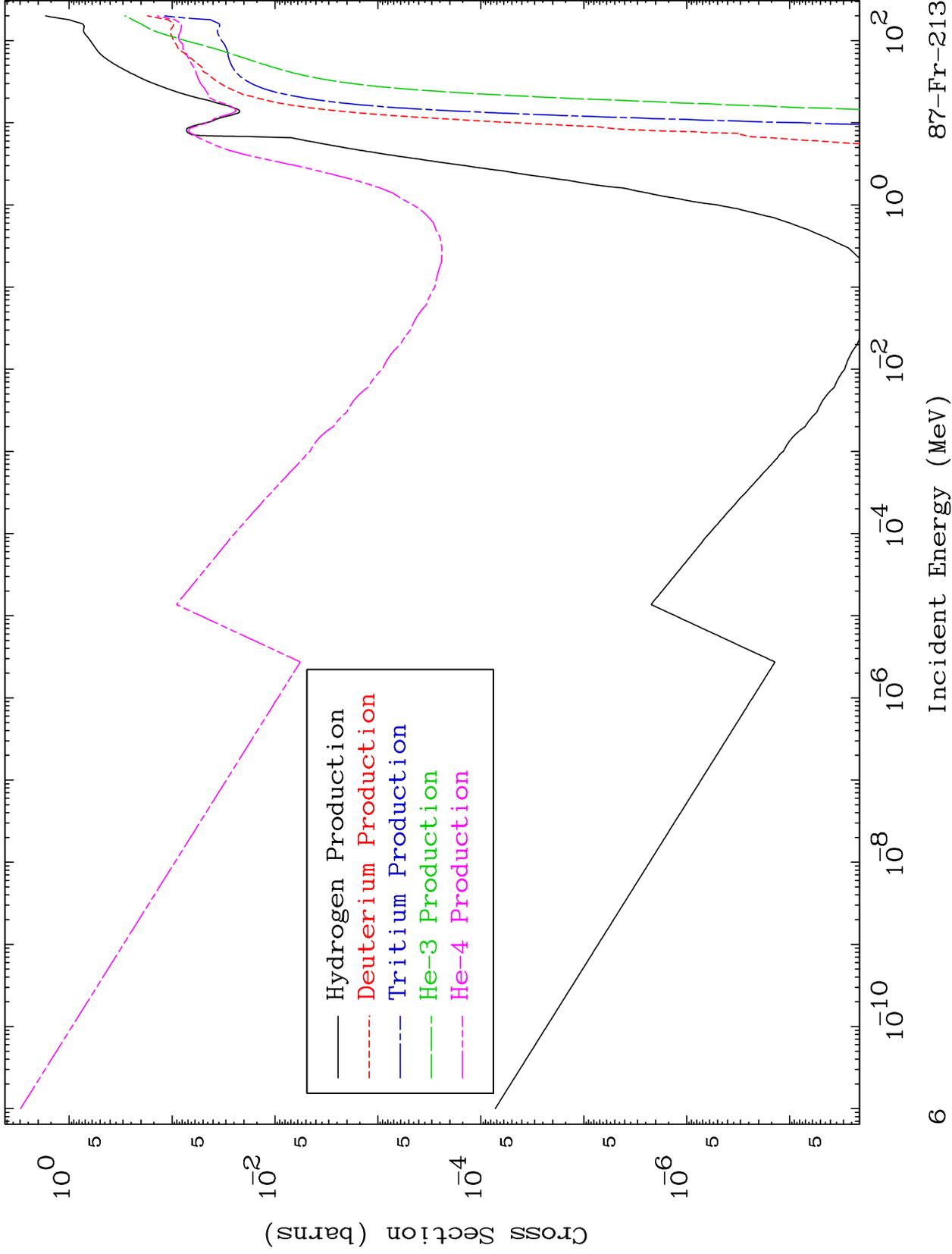




MAT 8728

Particle Production
293 Kelvin Cross Sections

87-Fr-213

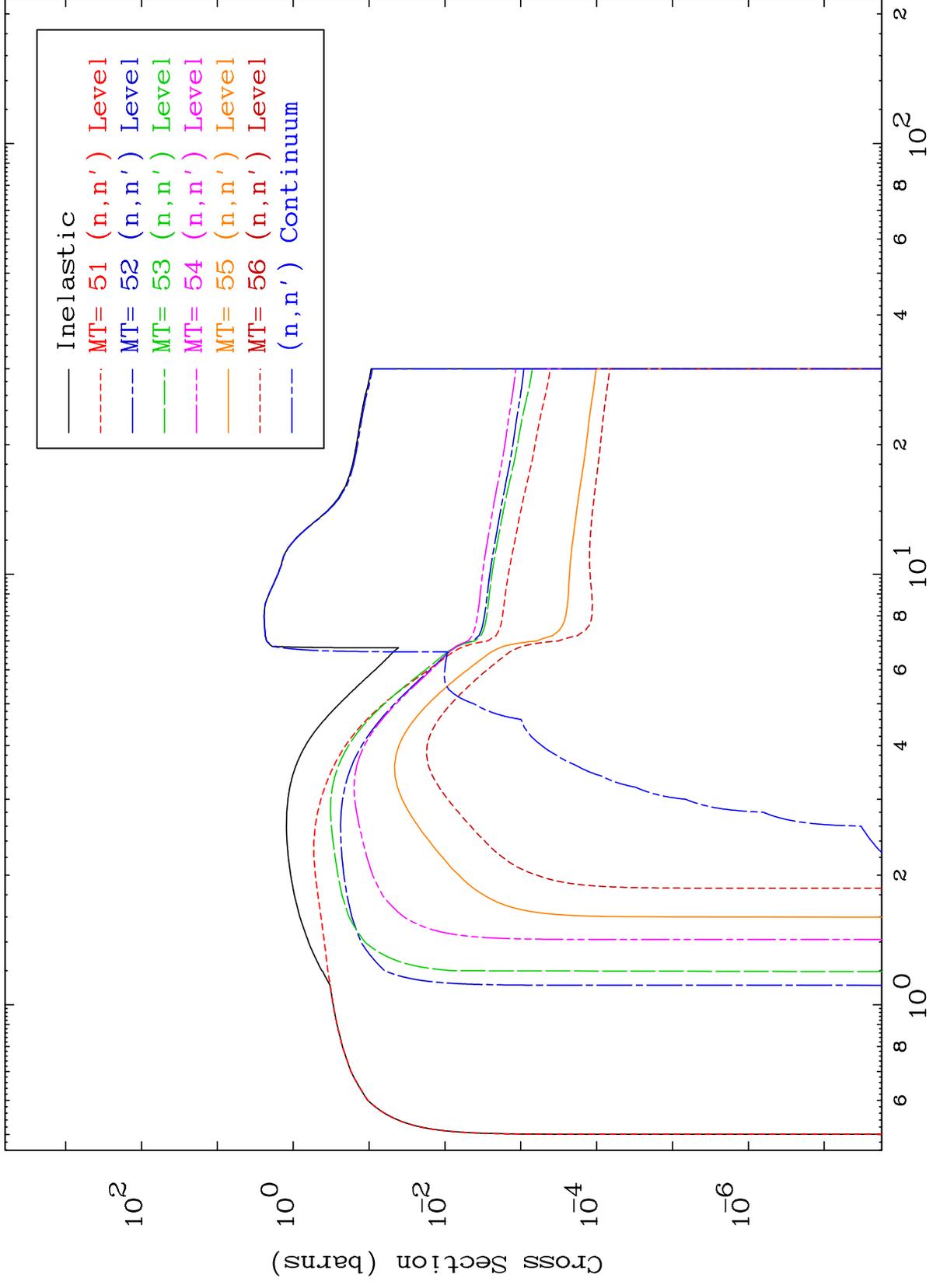


MAT 8728

(n,n') Level

87-Fr-213

293 Kelvin Cross Sections



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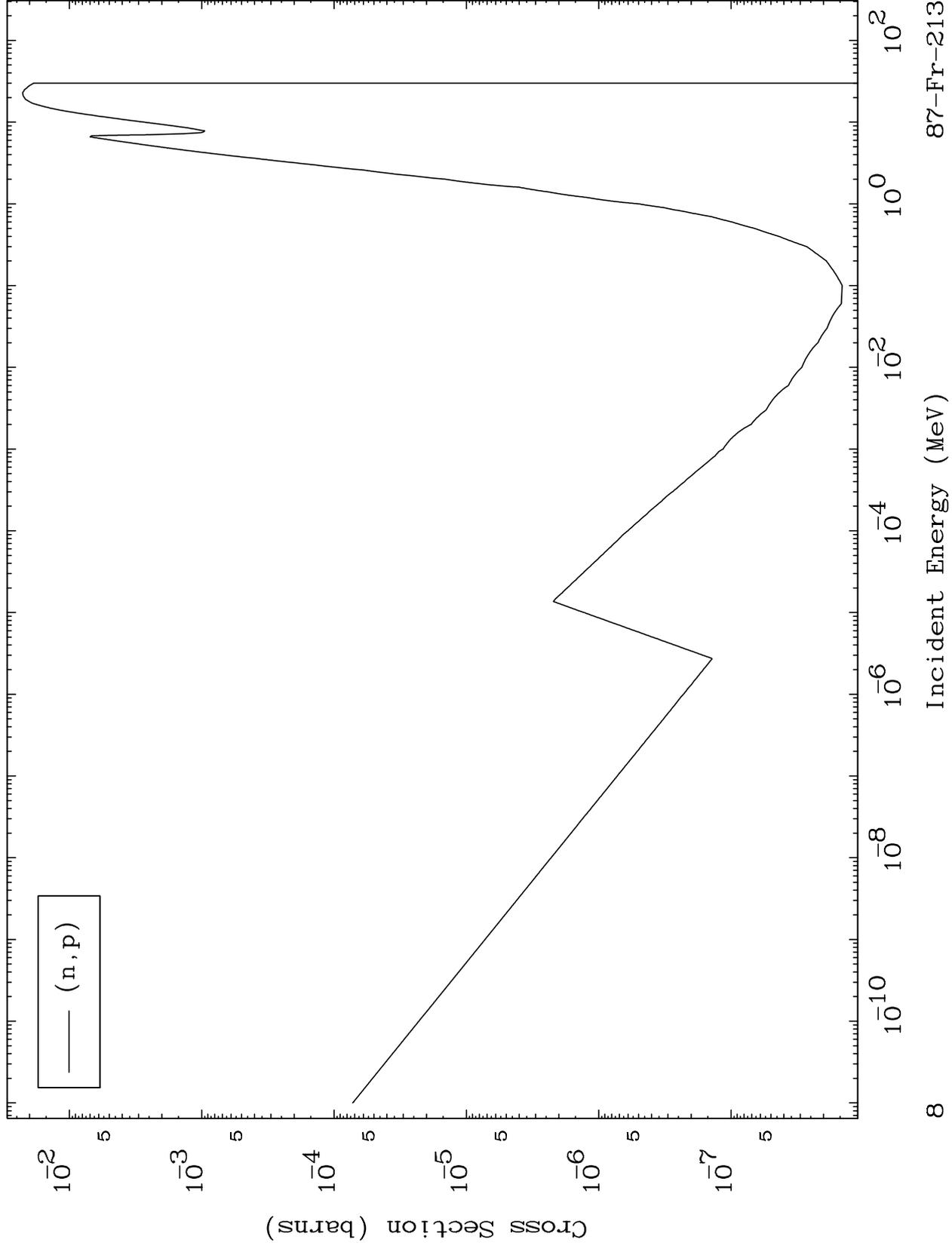
Incident Energy (MeV)

87-Fr-213

MAT 8728

(n,p) Levels
293 Kelvin Cross Sections

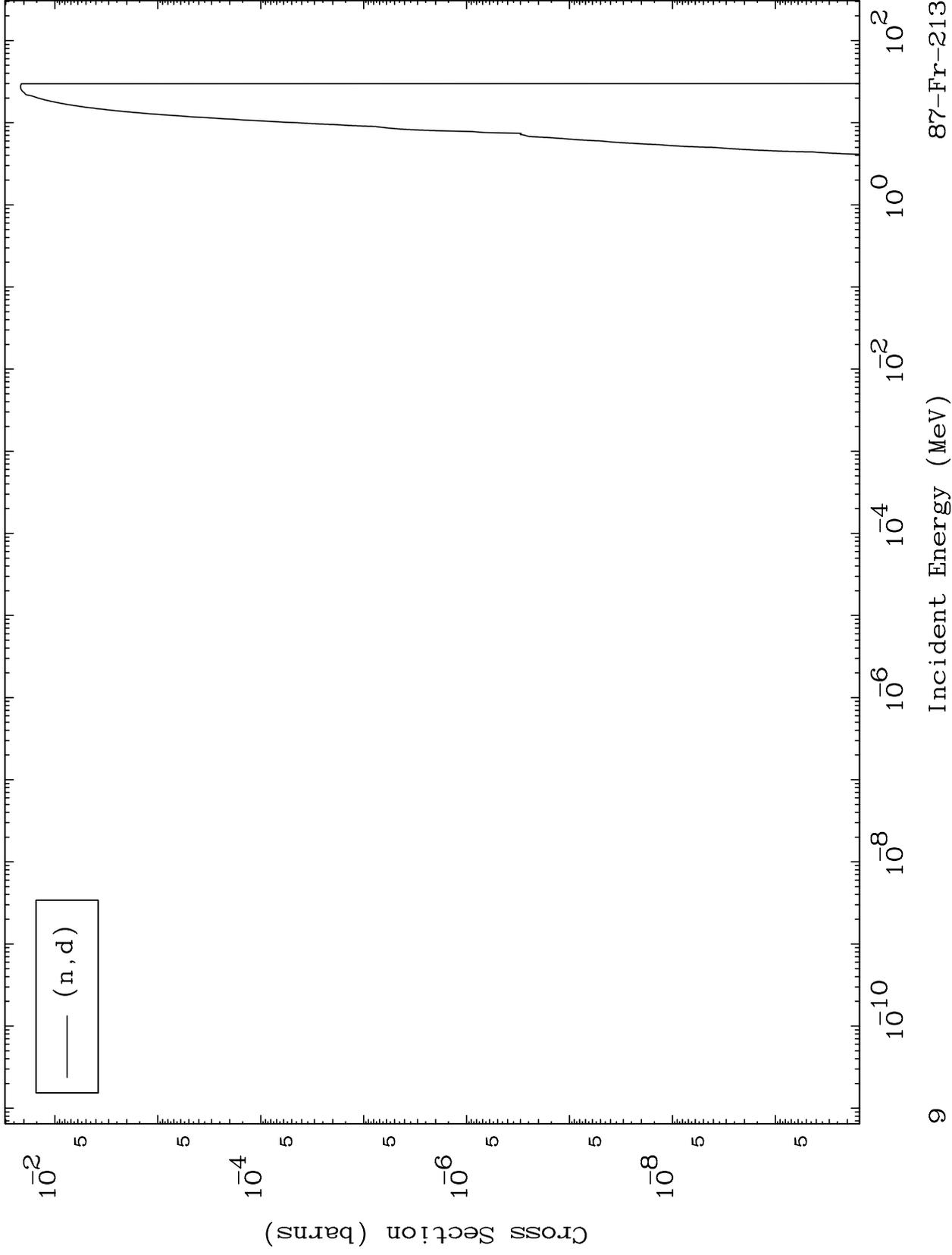
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MAT 8728

(n,d) Levels
293 Kelvin Cross Sections

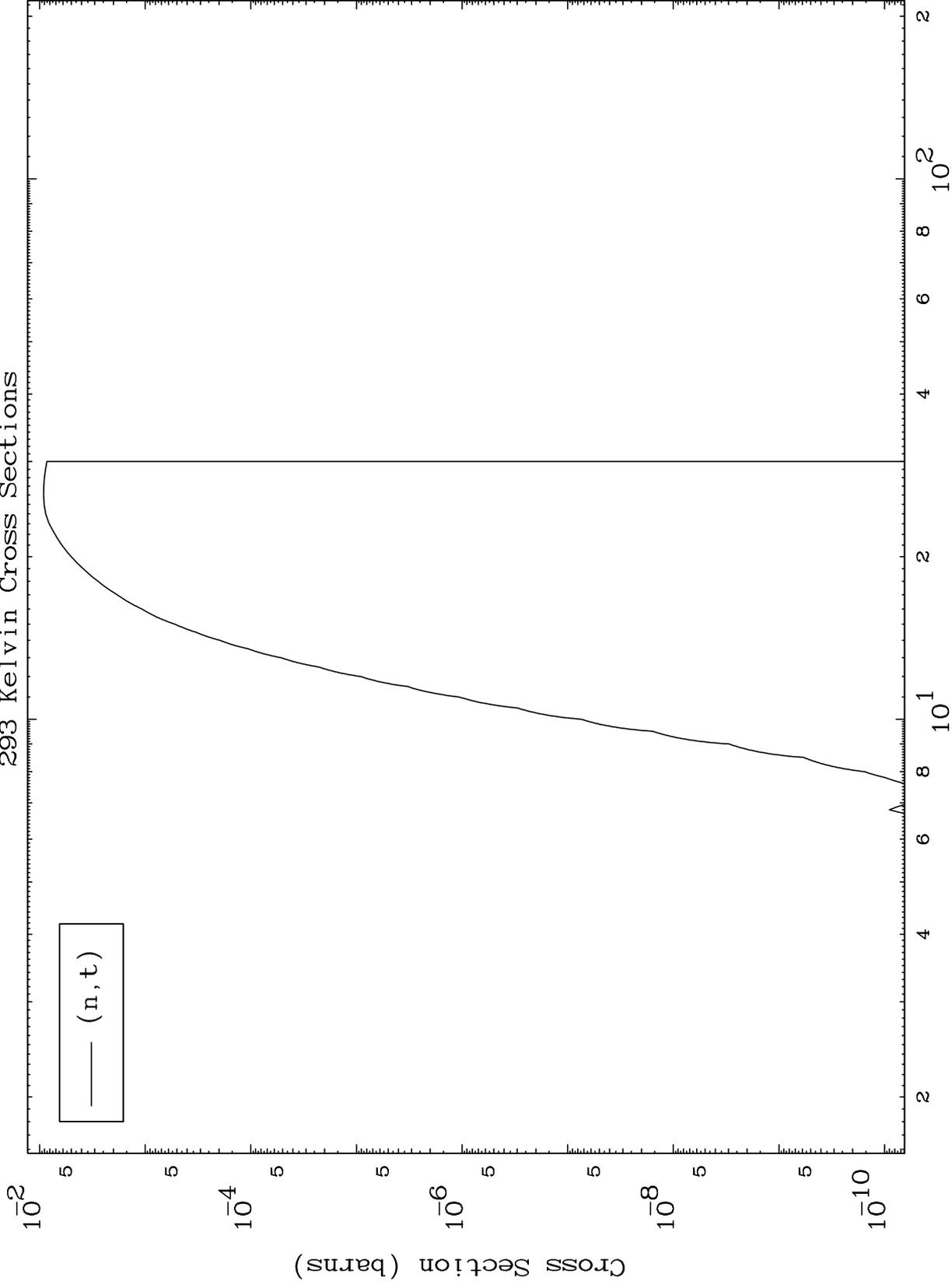
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MAT 8728

(n,t) Levels
293 Kelvin Cross Sections

87-Fr-213



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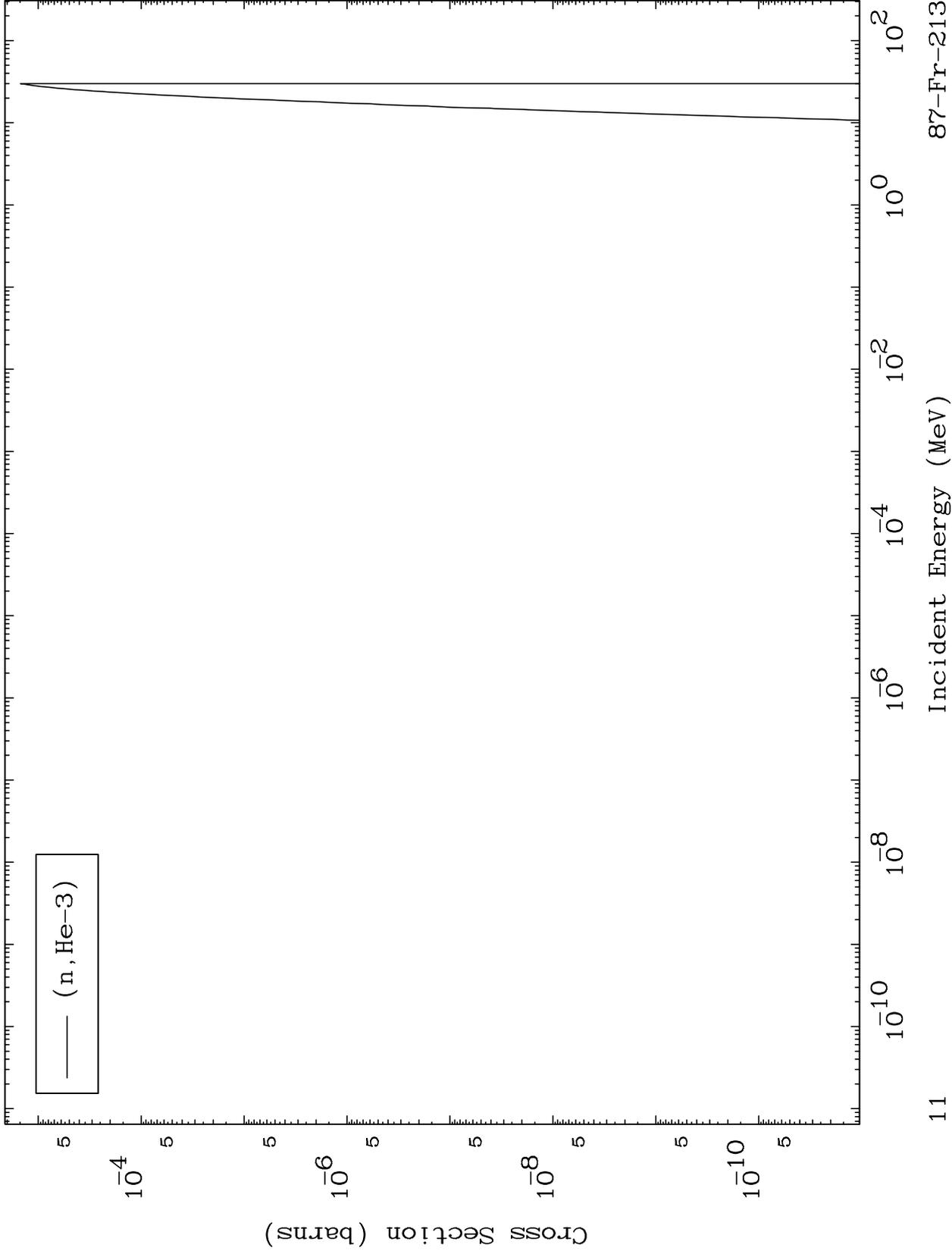
Incident Energy (MeV)

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MAT 8728

(n,He3) Levels
293 Kelvin Cross Sections

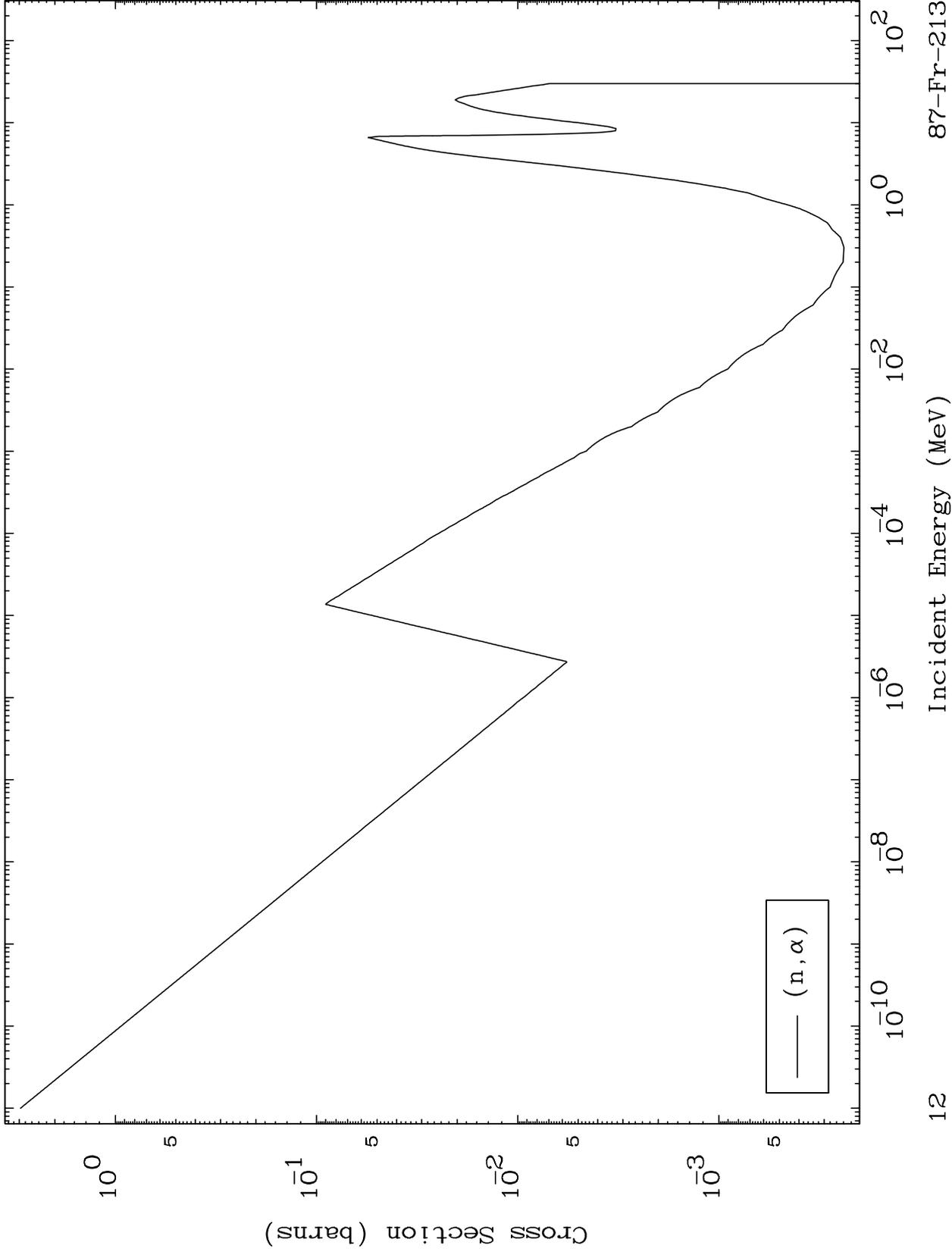
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MAT 8728

(n,α) Levels
293 Kelvin Cross Sections

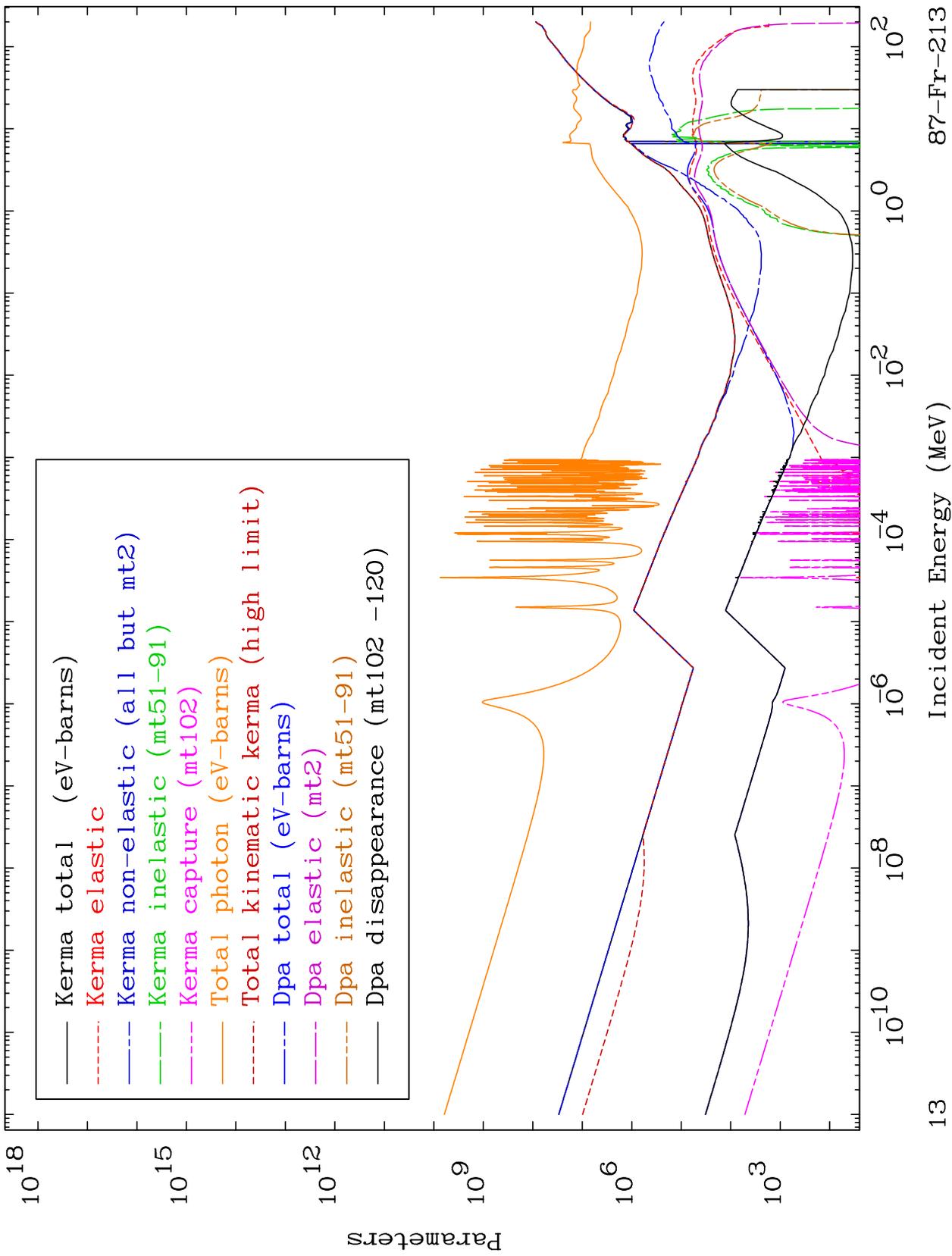
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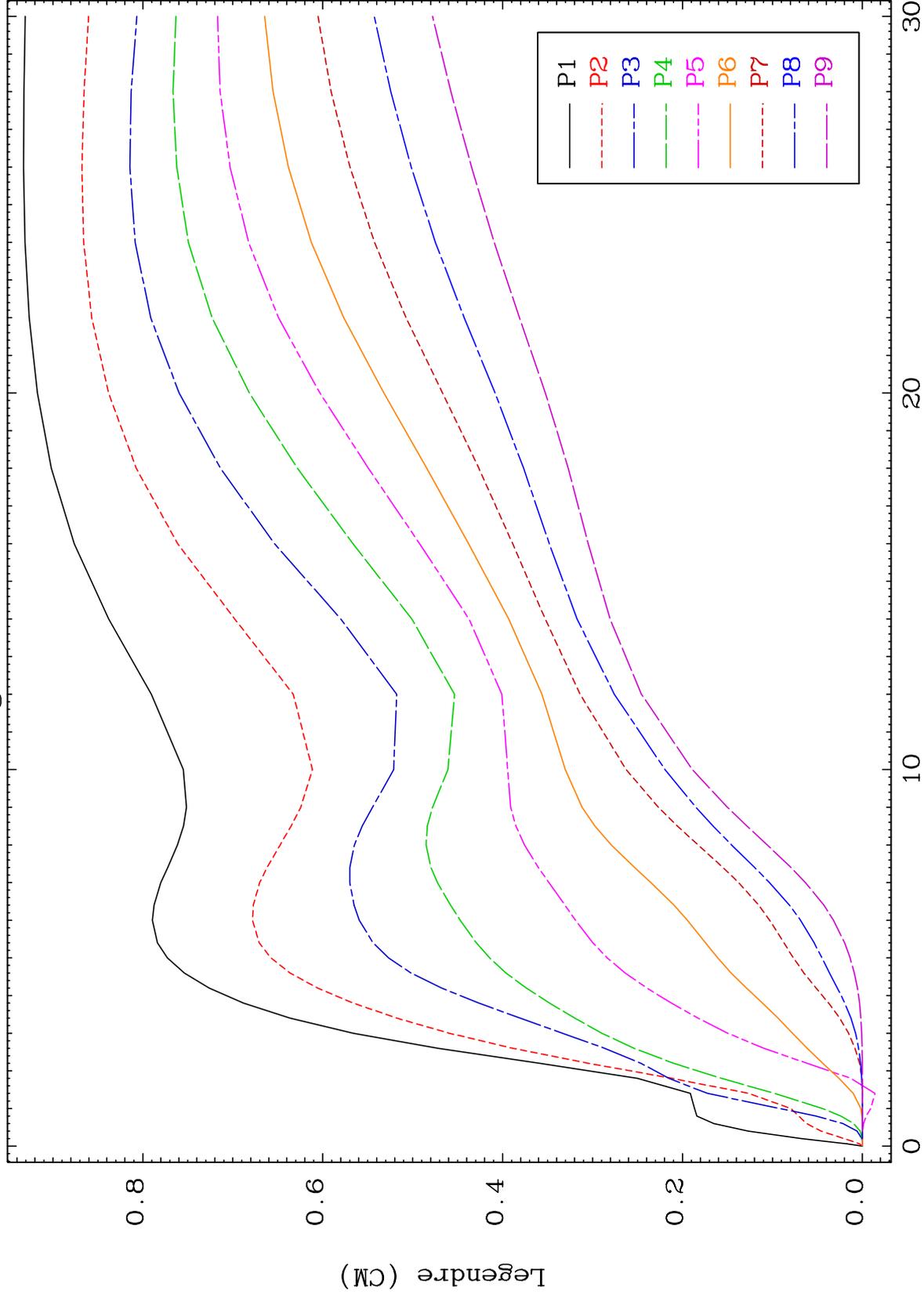
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MAT 8728

Elastic Legendre Coefficients

87-Fr-213



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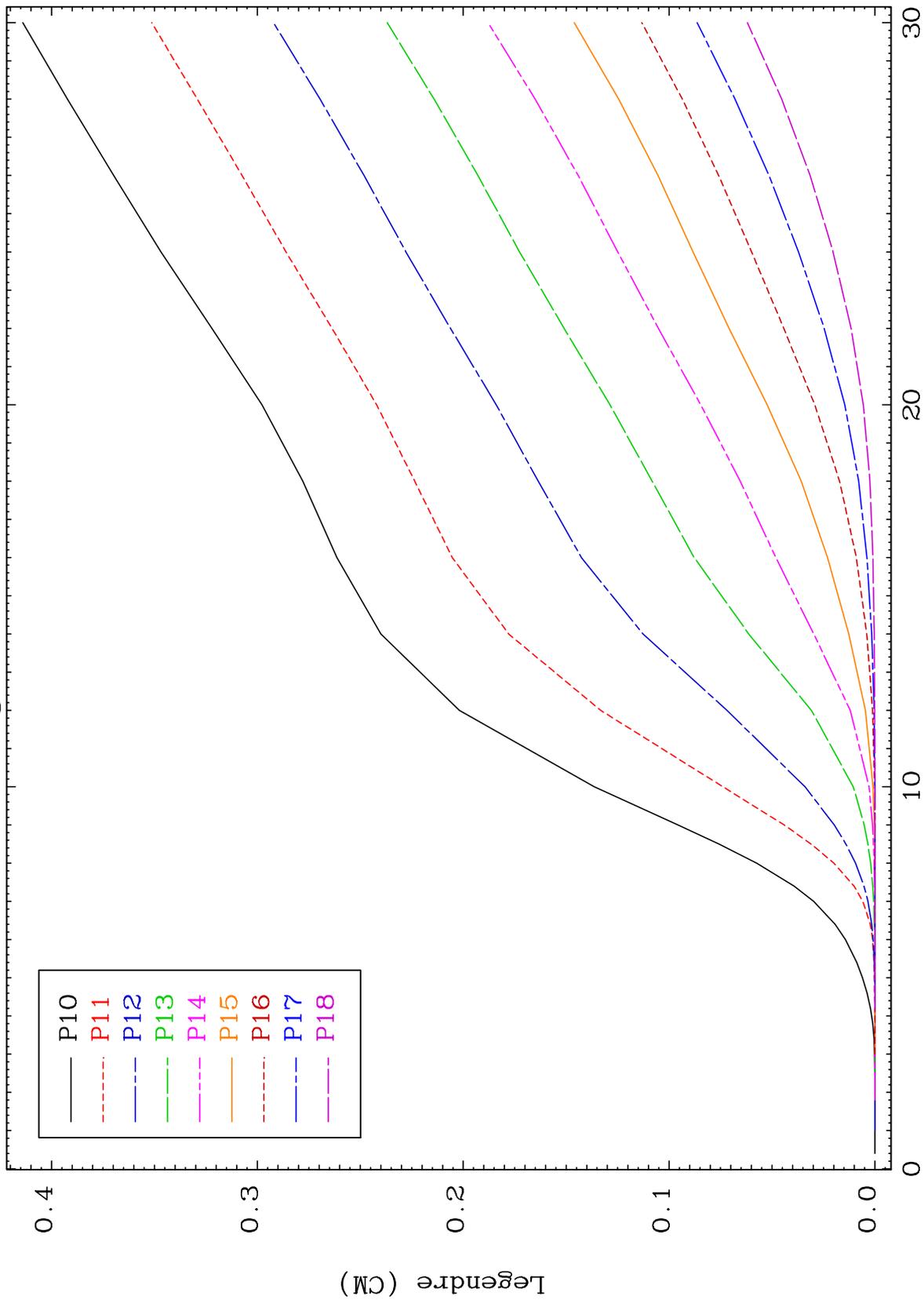
Incident Energy (MeV)

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MAT 8728

Elastic Legendre Coefficients

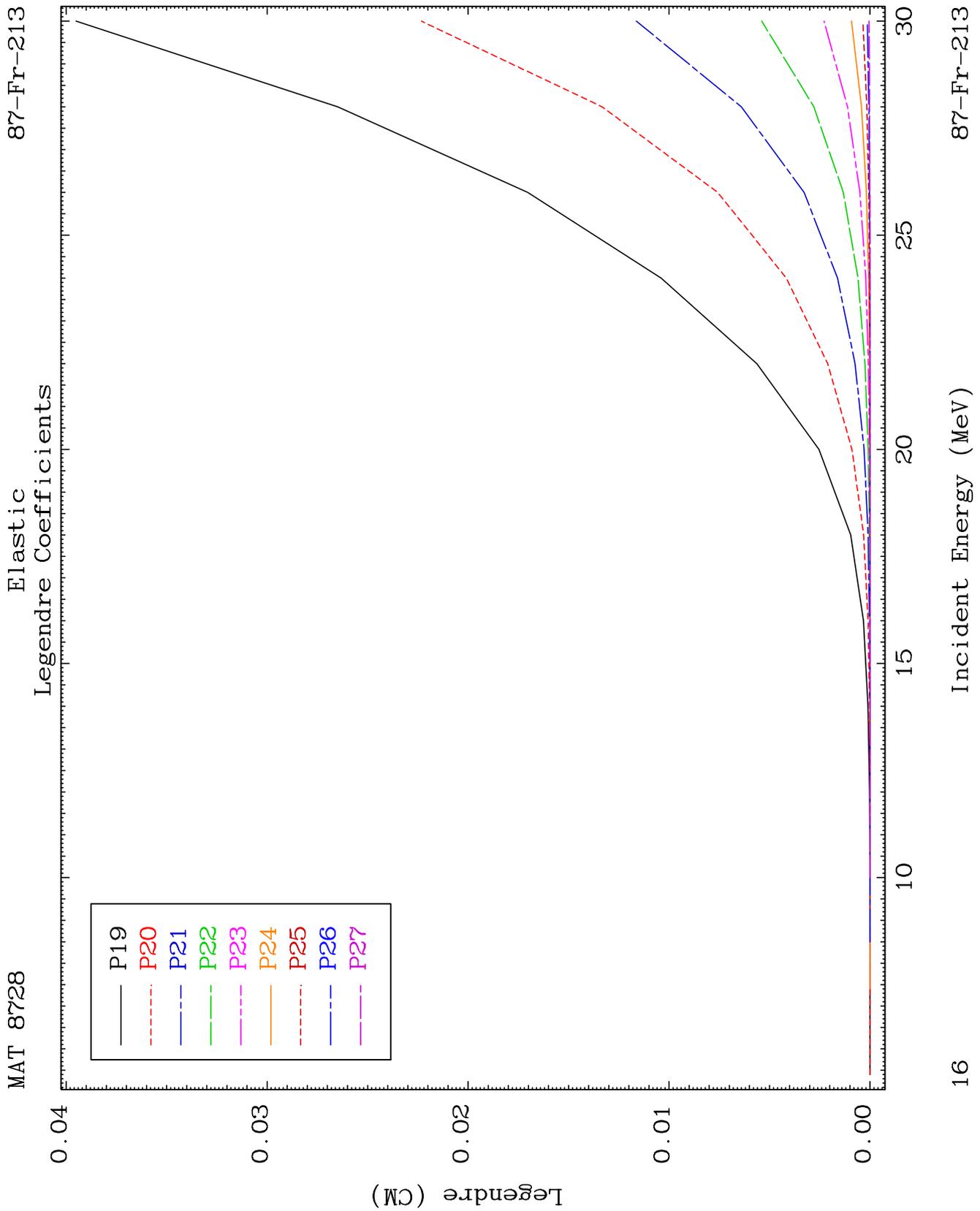
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Incident Energy (MeV)

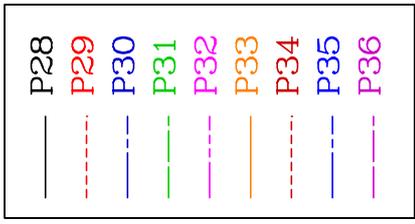
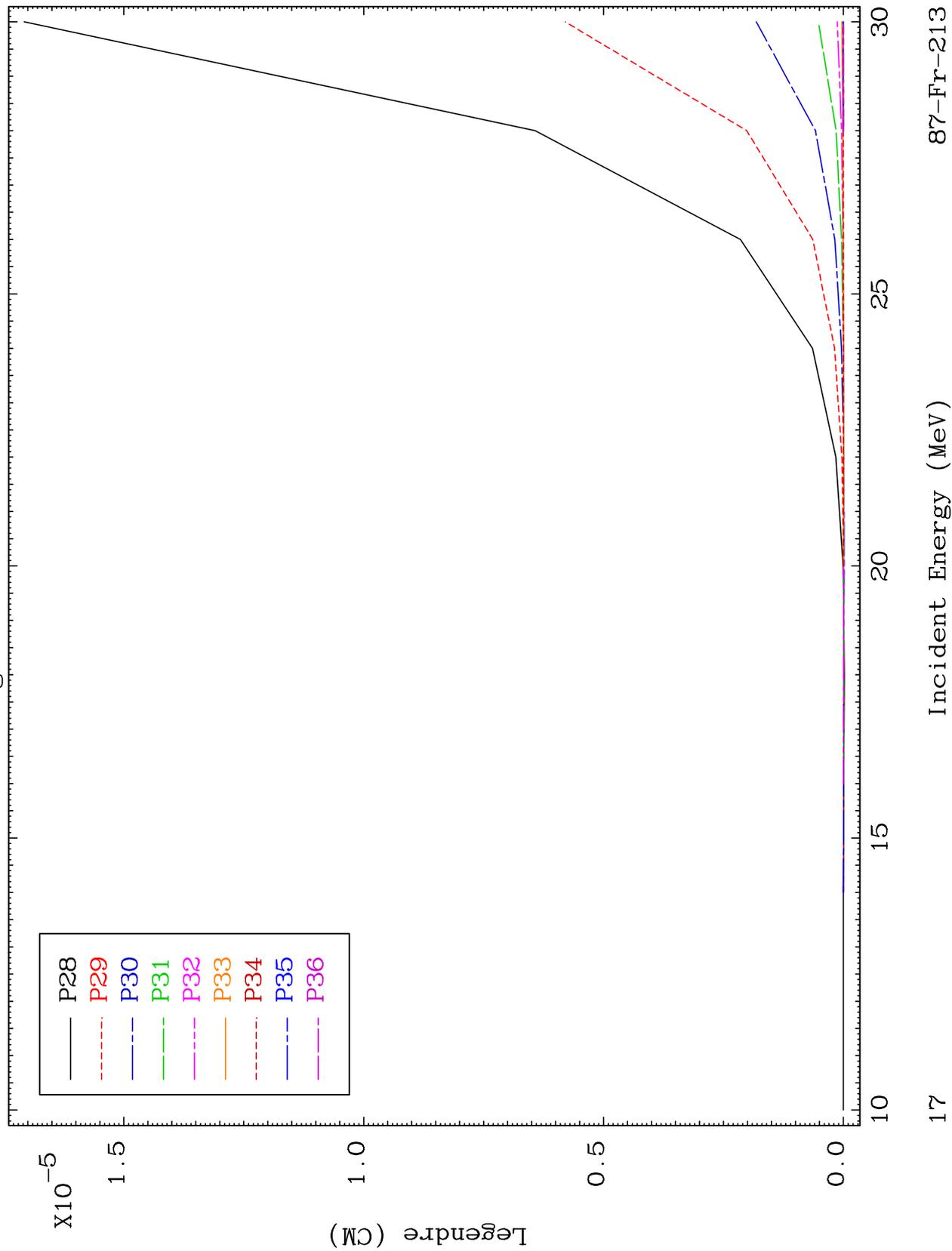
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MAT 8728

Elastic
Legendre Coefficients

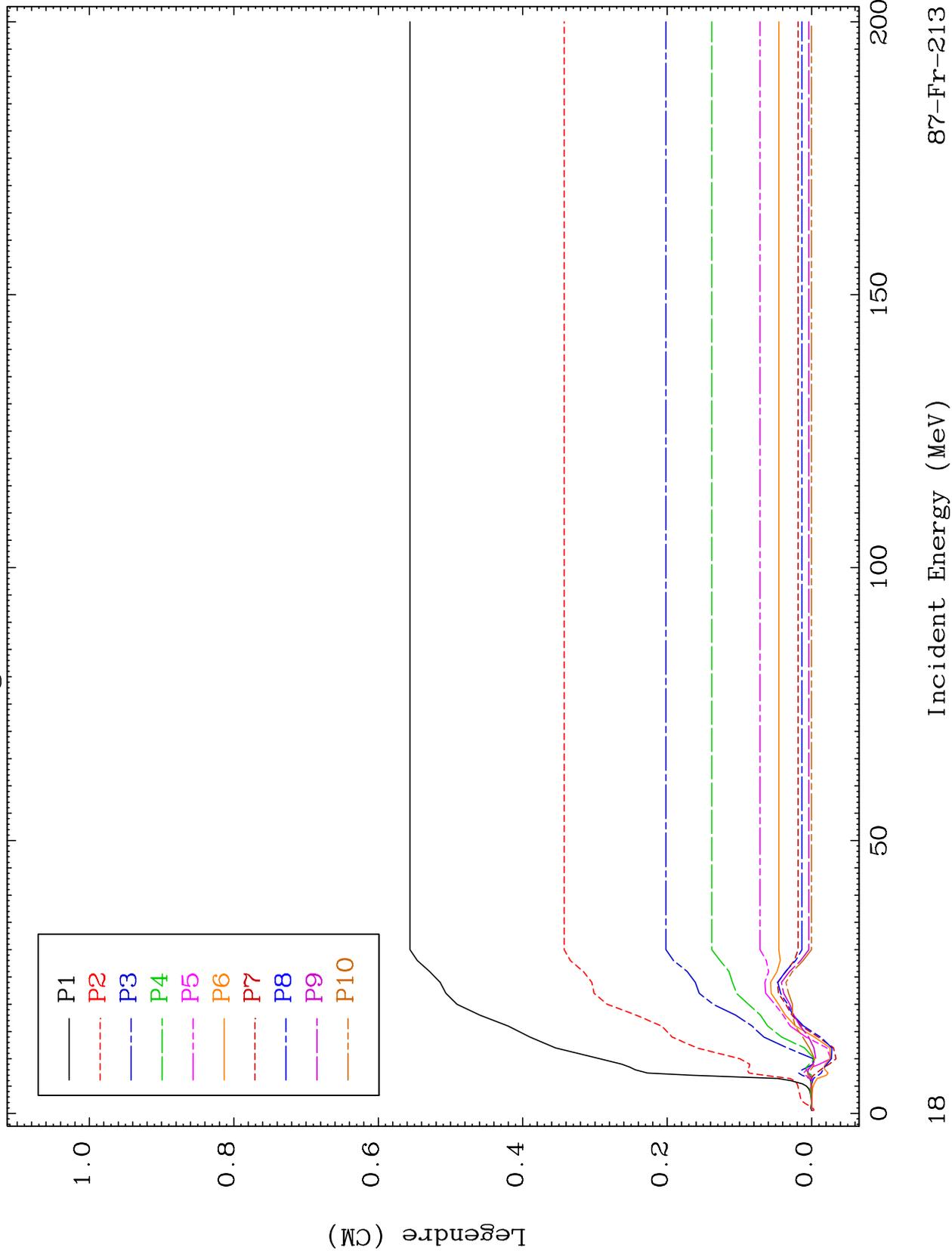
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MAT 8728

MT= 51 (n,n') Level
Legendre Coefficients

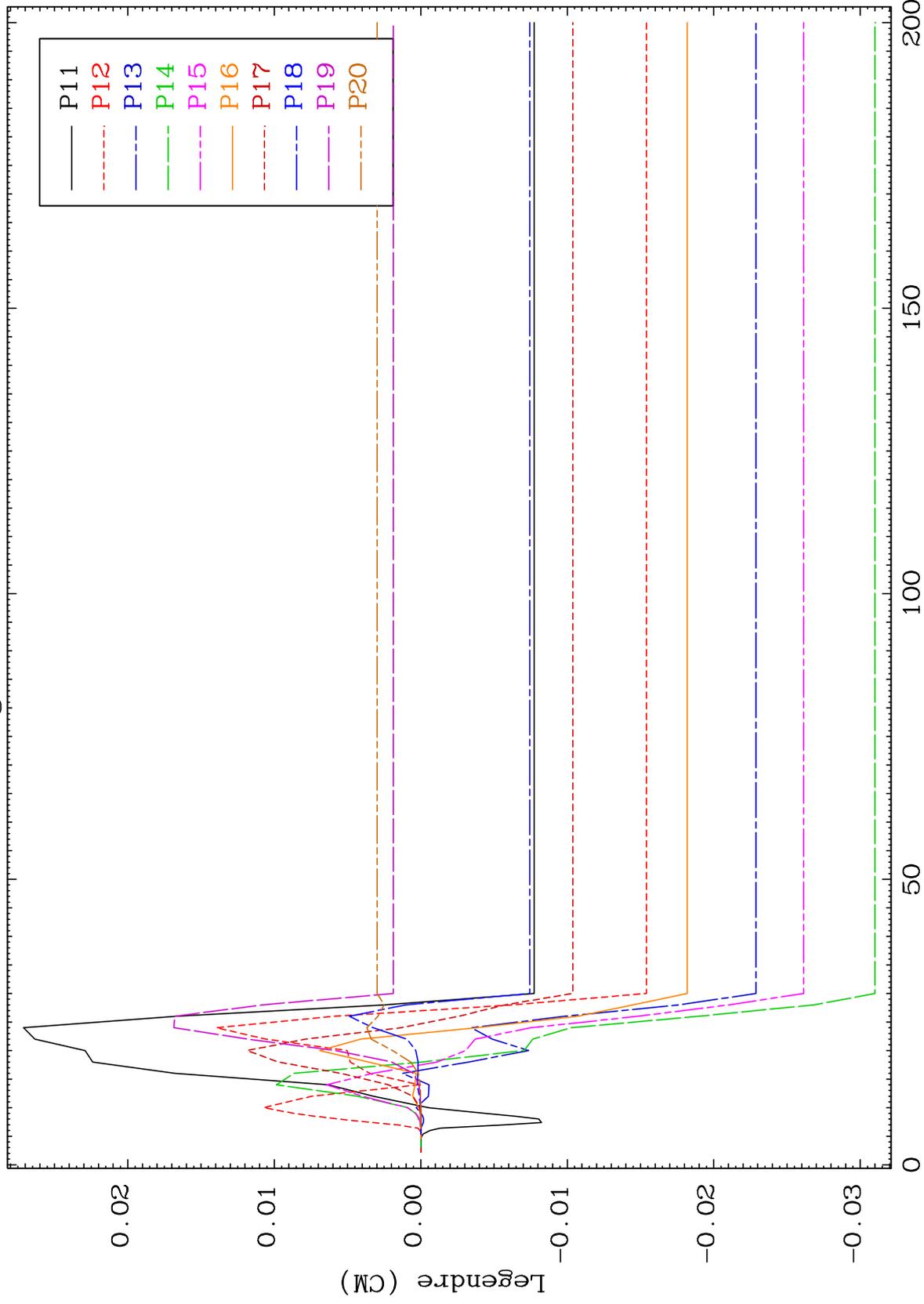
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MAT 8728

MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-213



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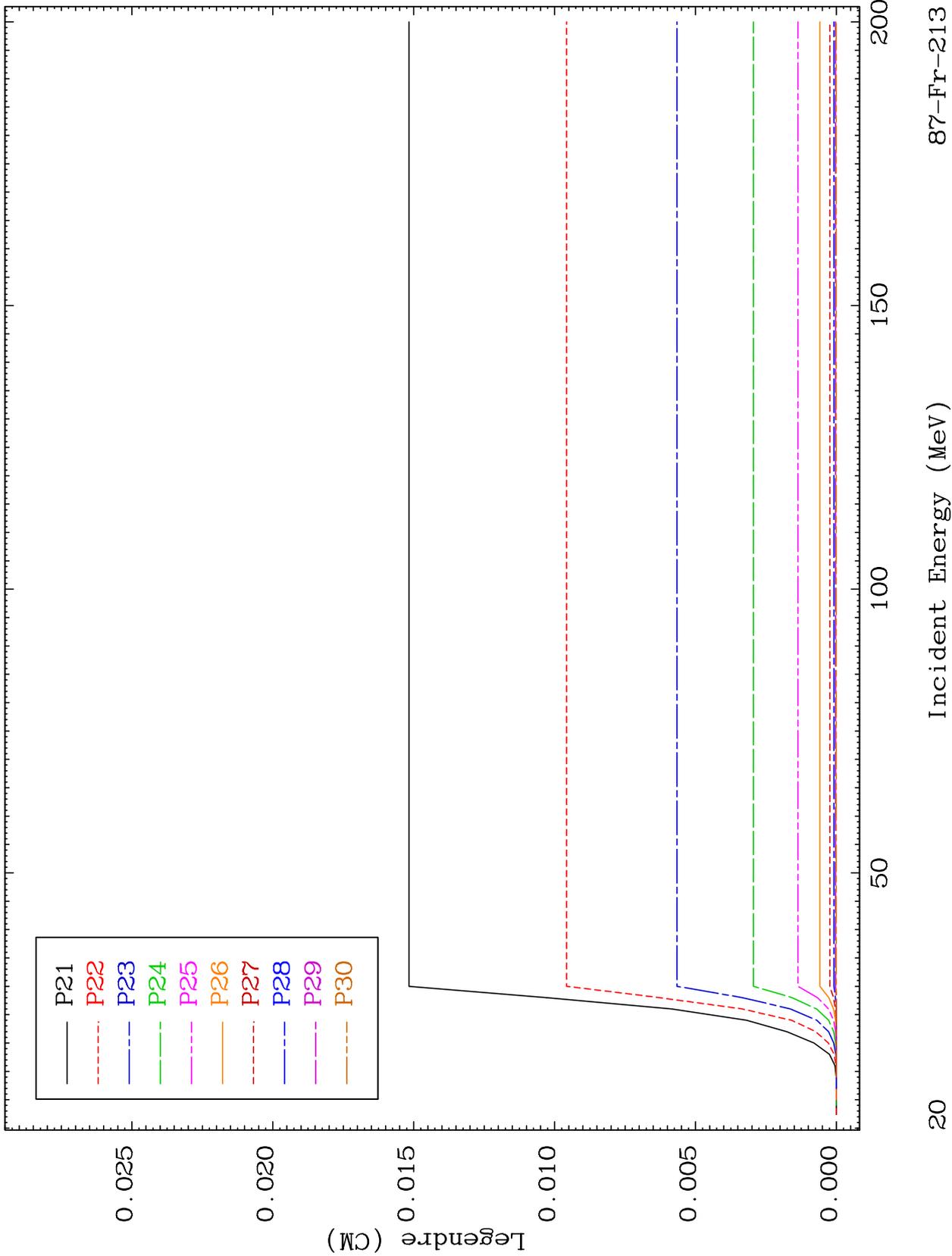
Incident Energy (MeV)

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MAT 8728

MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-213



87-Fr-213

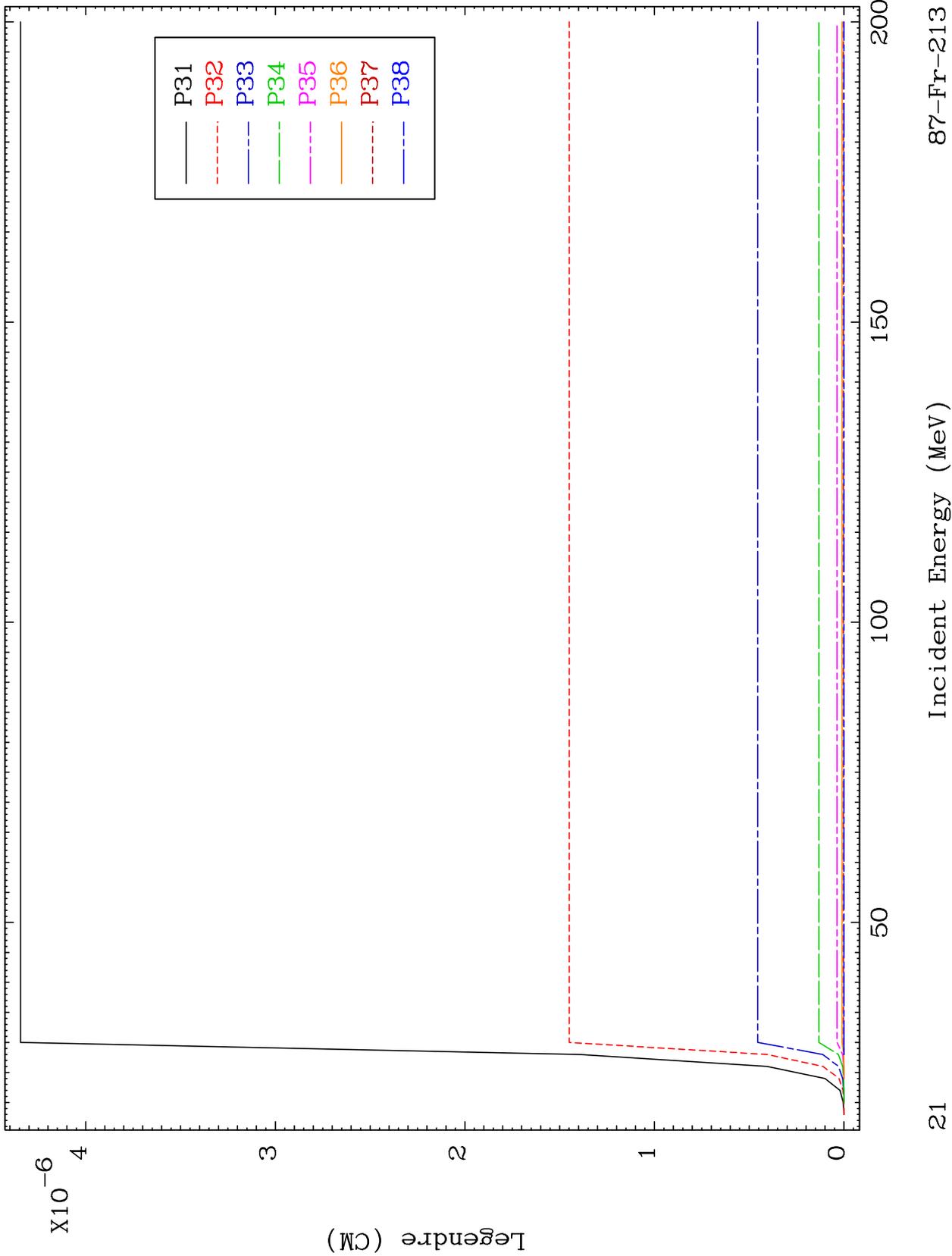
Incident Energy (MeV)

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MAT 8728

MT= 51 (n,n') Level
Legendre Coefficients

87-Fr-213



21

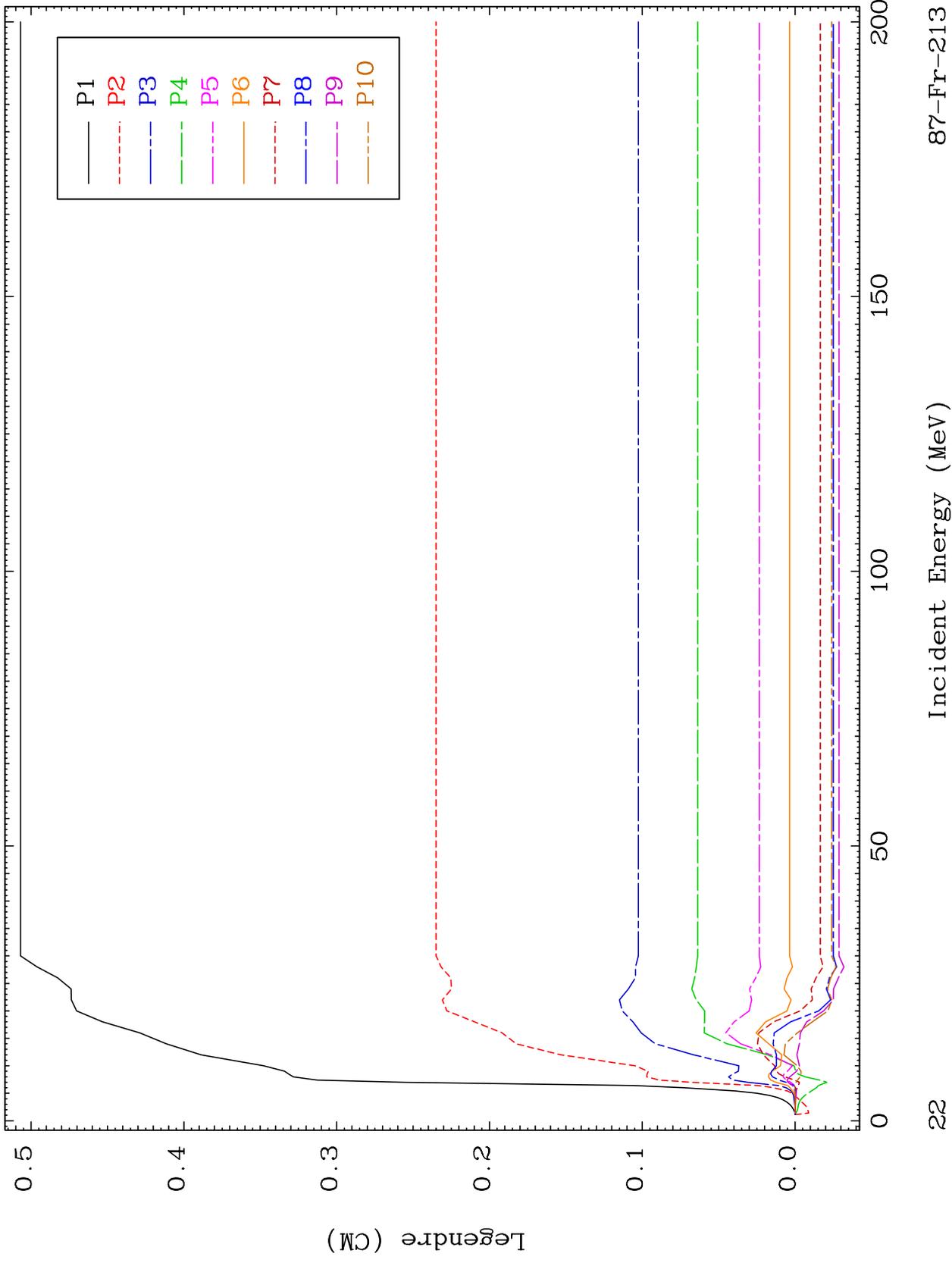
Incident Energy (MeV)

87-Fr-213

MAT 8728

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-213



87-Fr-213

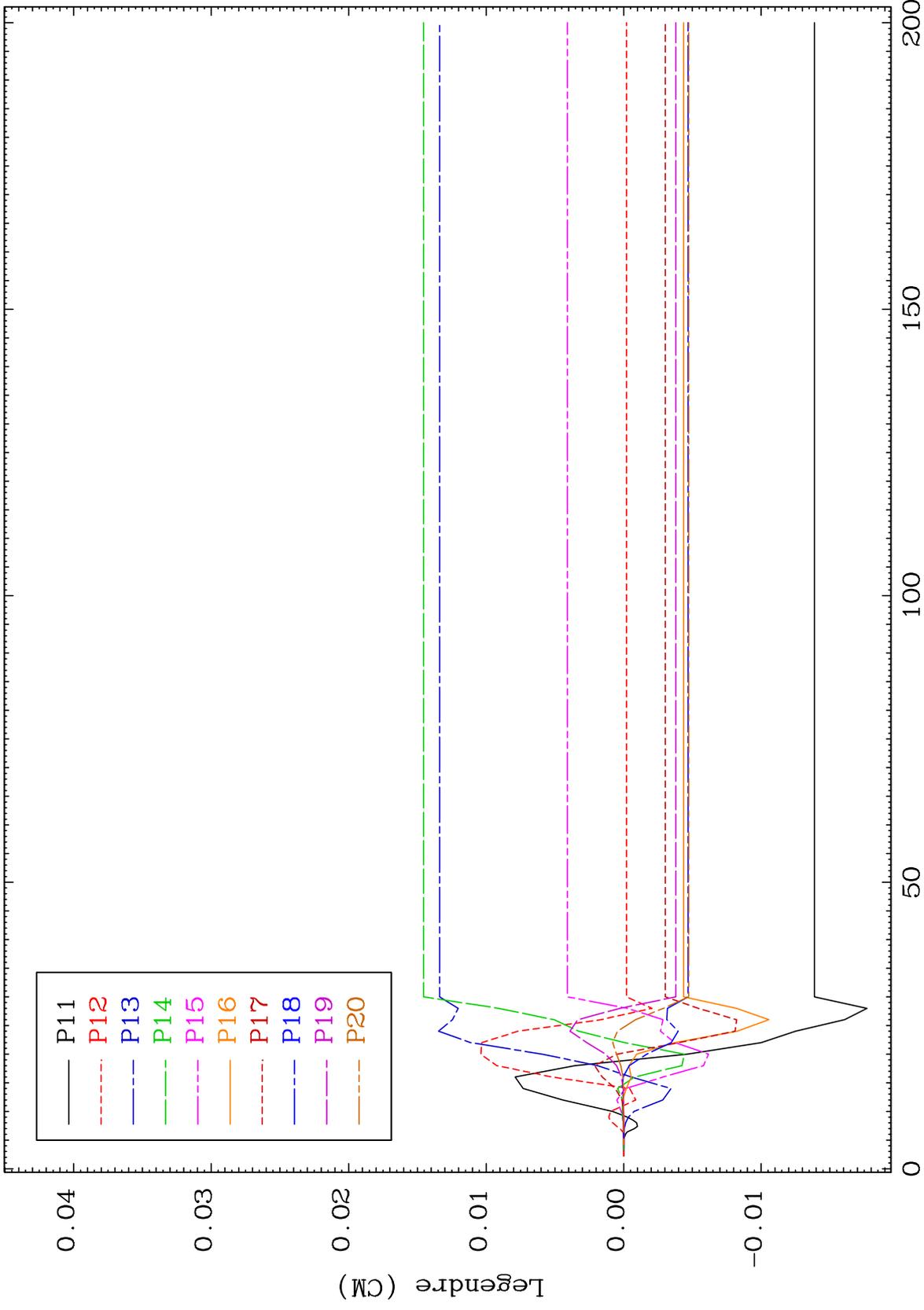
Incident Energy (MeV)

22

MAT 8728

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-213



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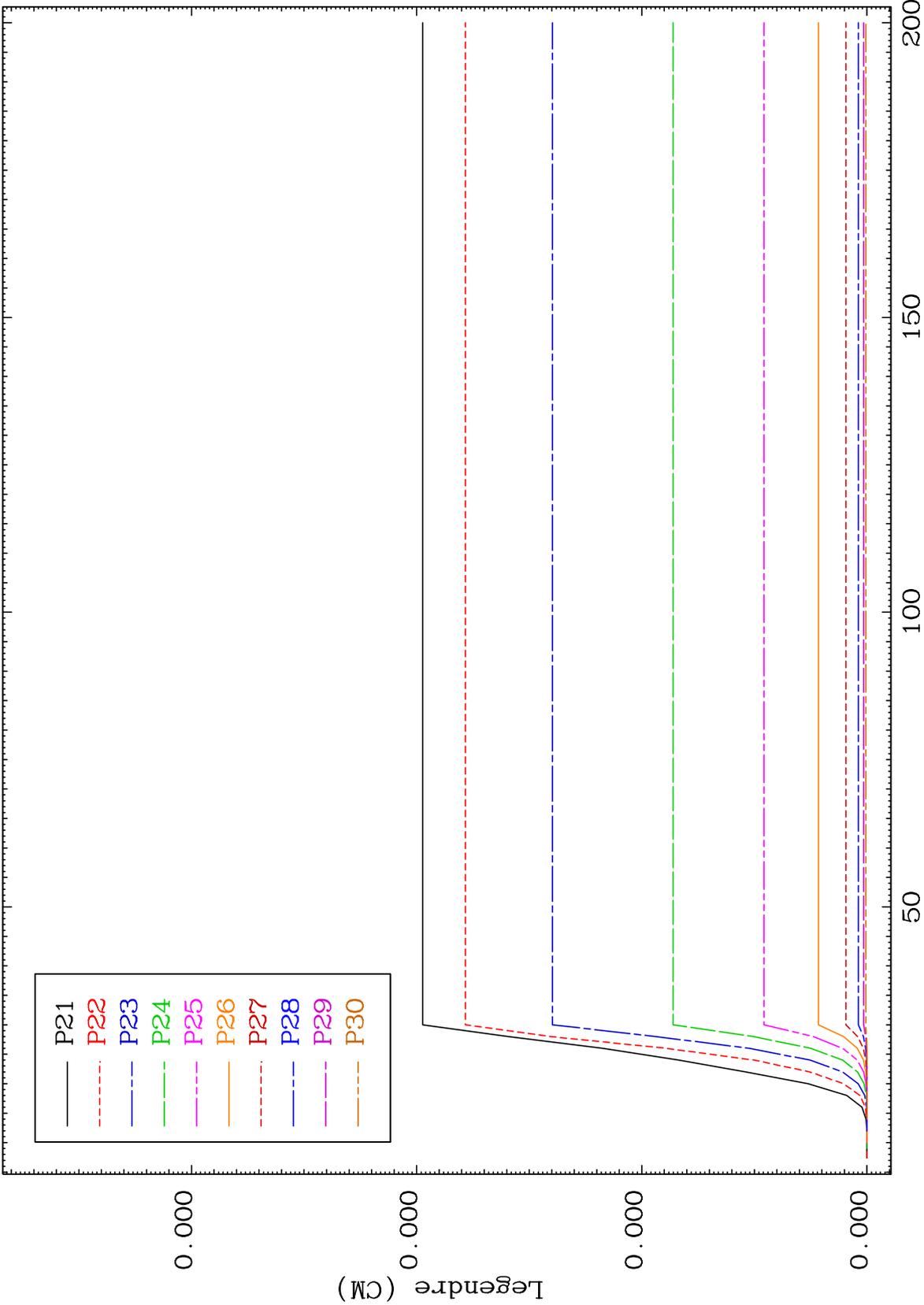
Incident Energy (MeV)

87-Fr-213

MAT 8728

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-213



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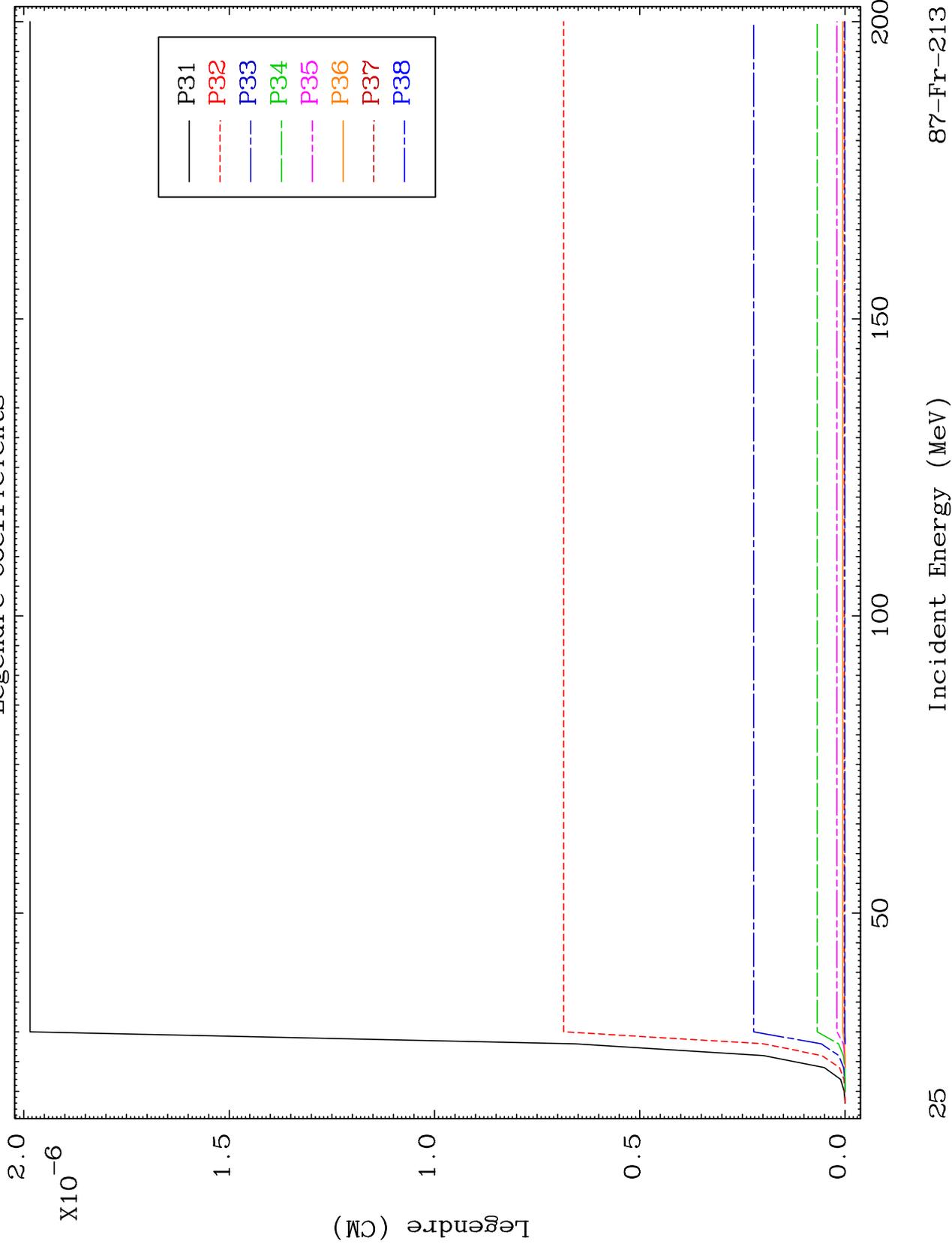
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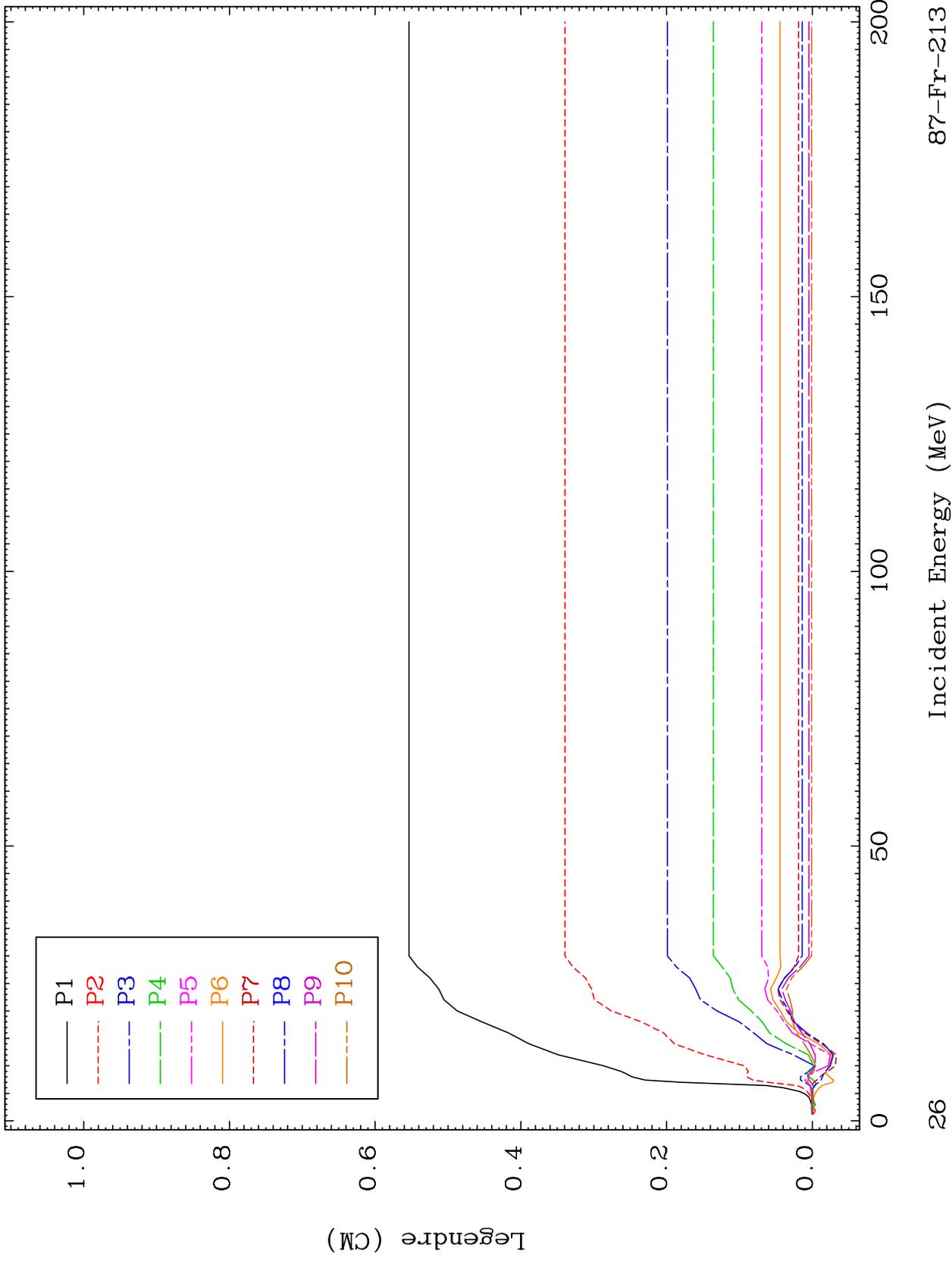
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MAT 8728

MT= 52 (n,n') Level
Legendre Coefficients

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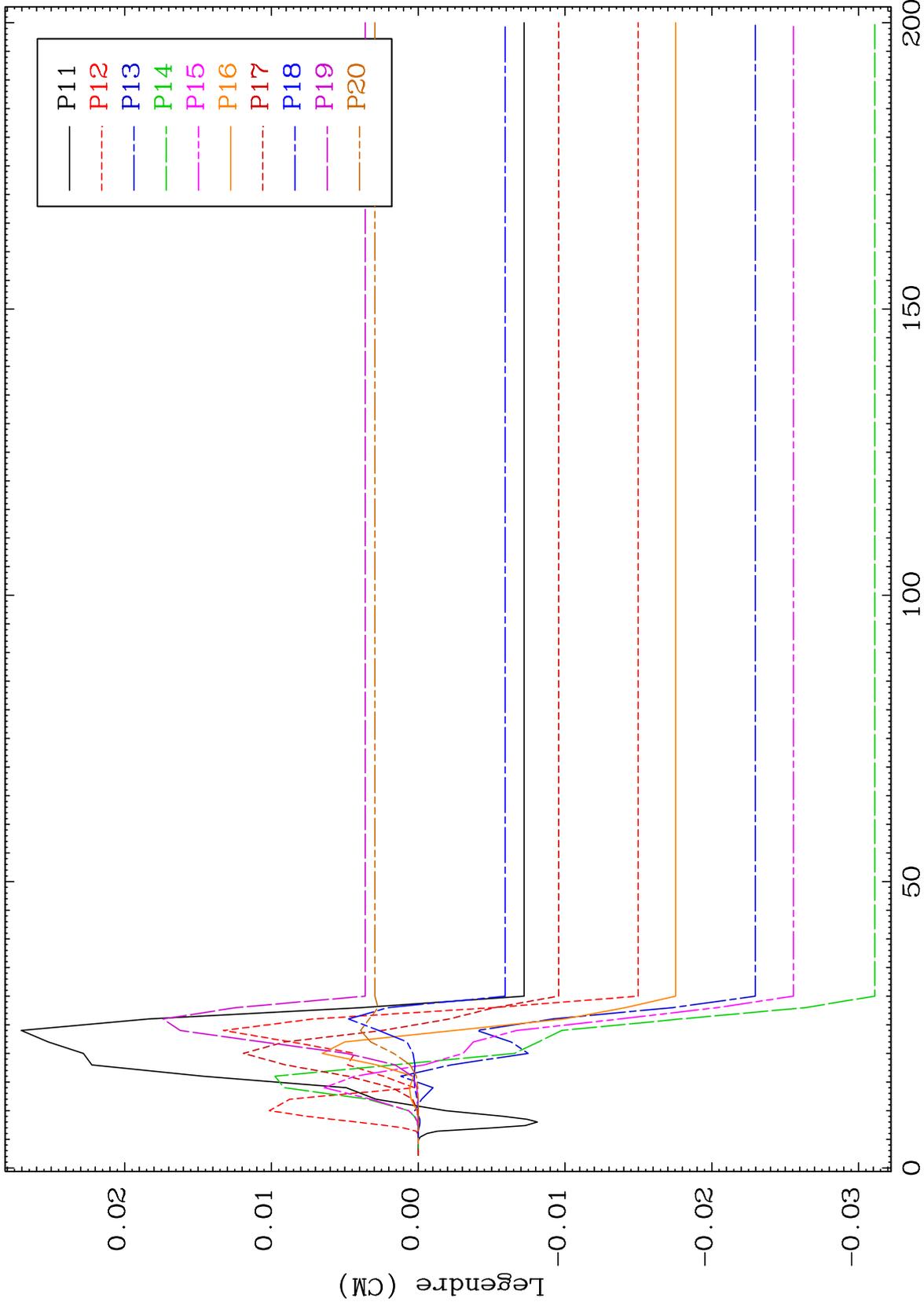




MAT 8728

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-213



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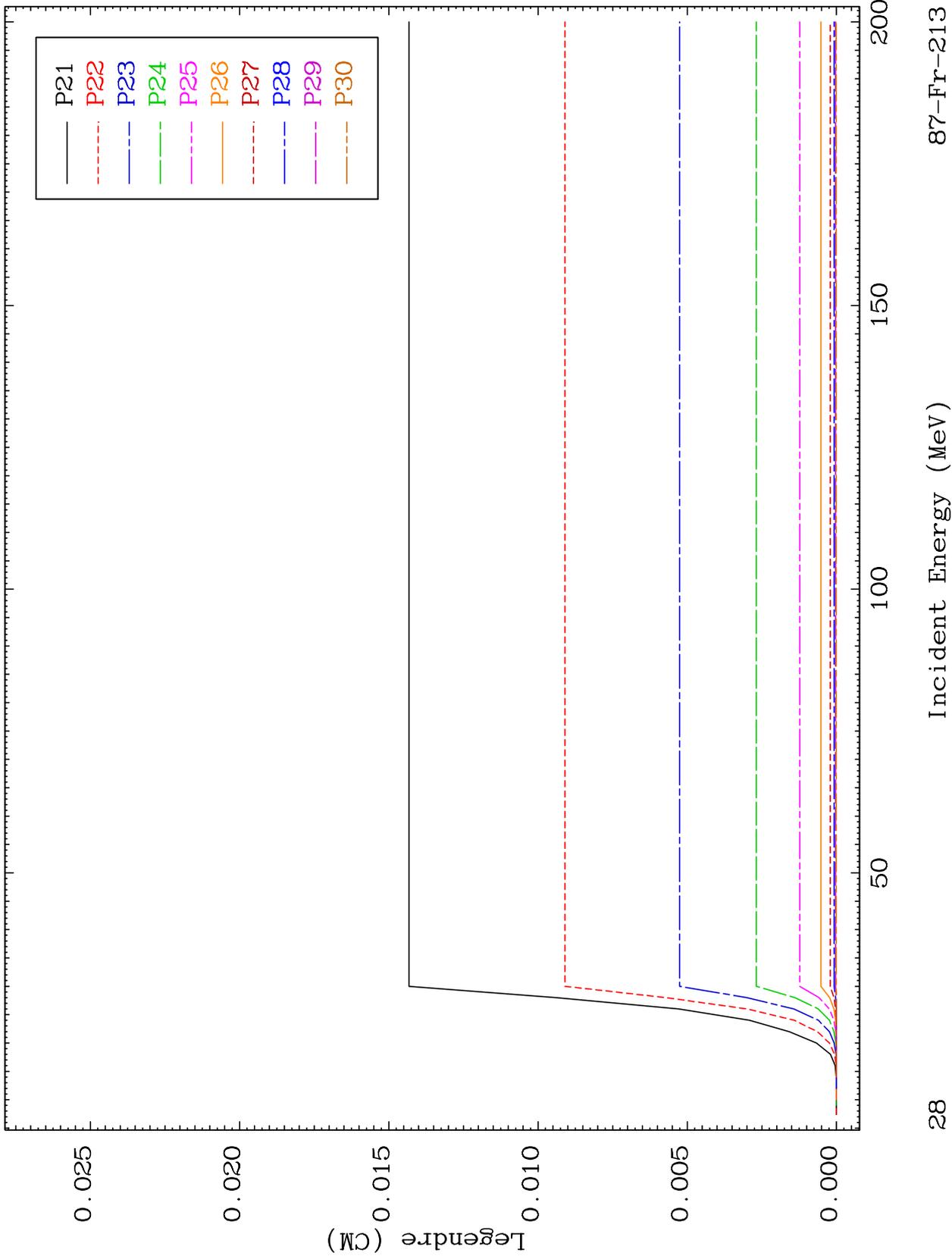
Incident Energy (MeV)

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MAT 8728

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-213



28

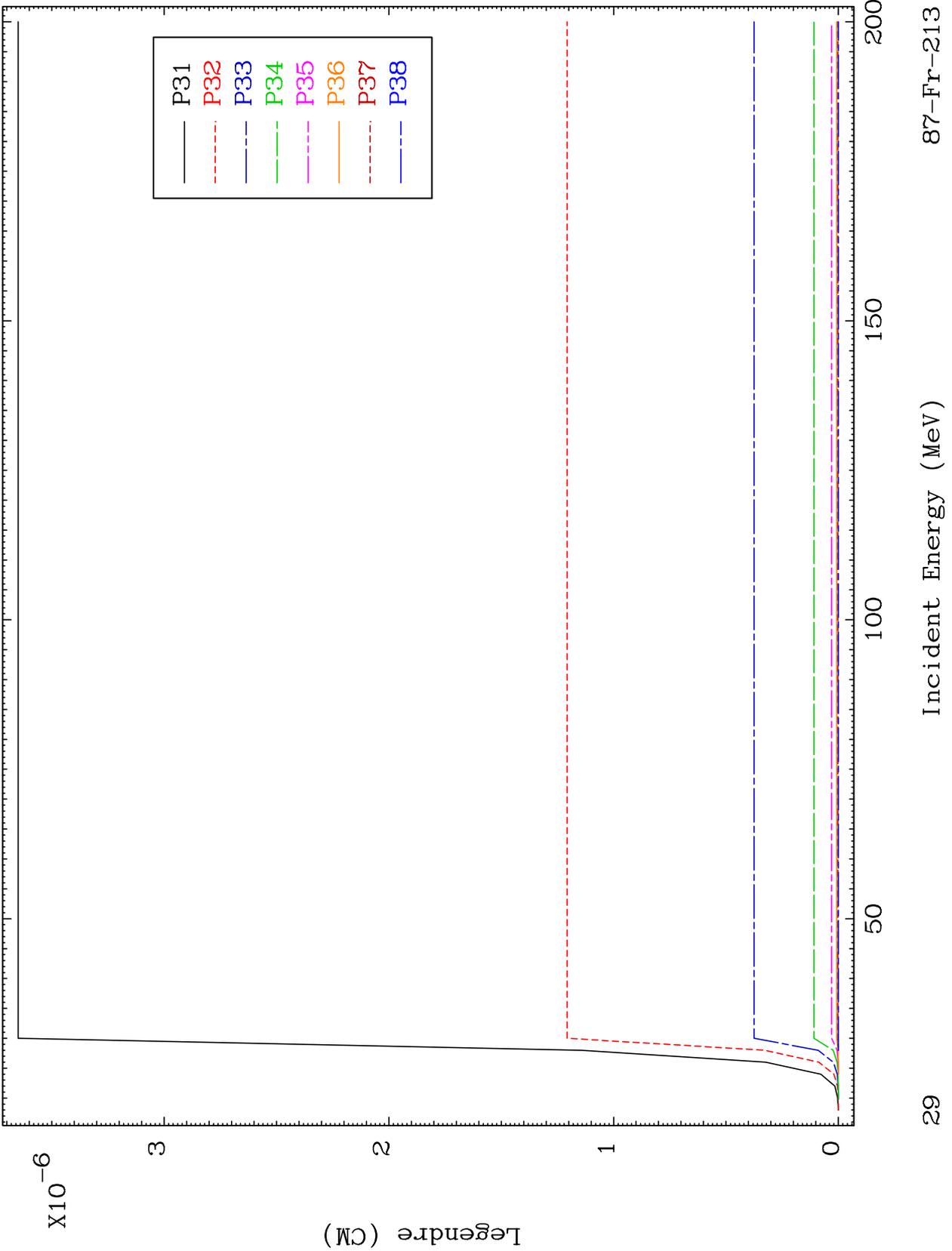
Incident Energy (MeV)

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MAT 8728

MT= 53 (n,n') Level
Legendre Coefficients

87-Fr-213



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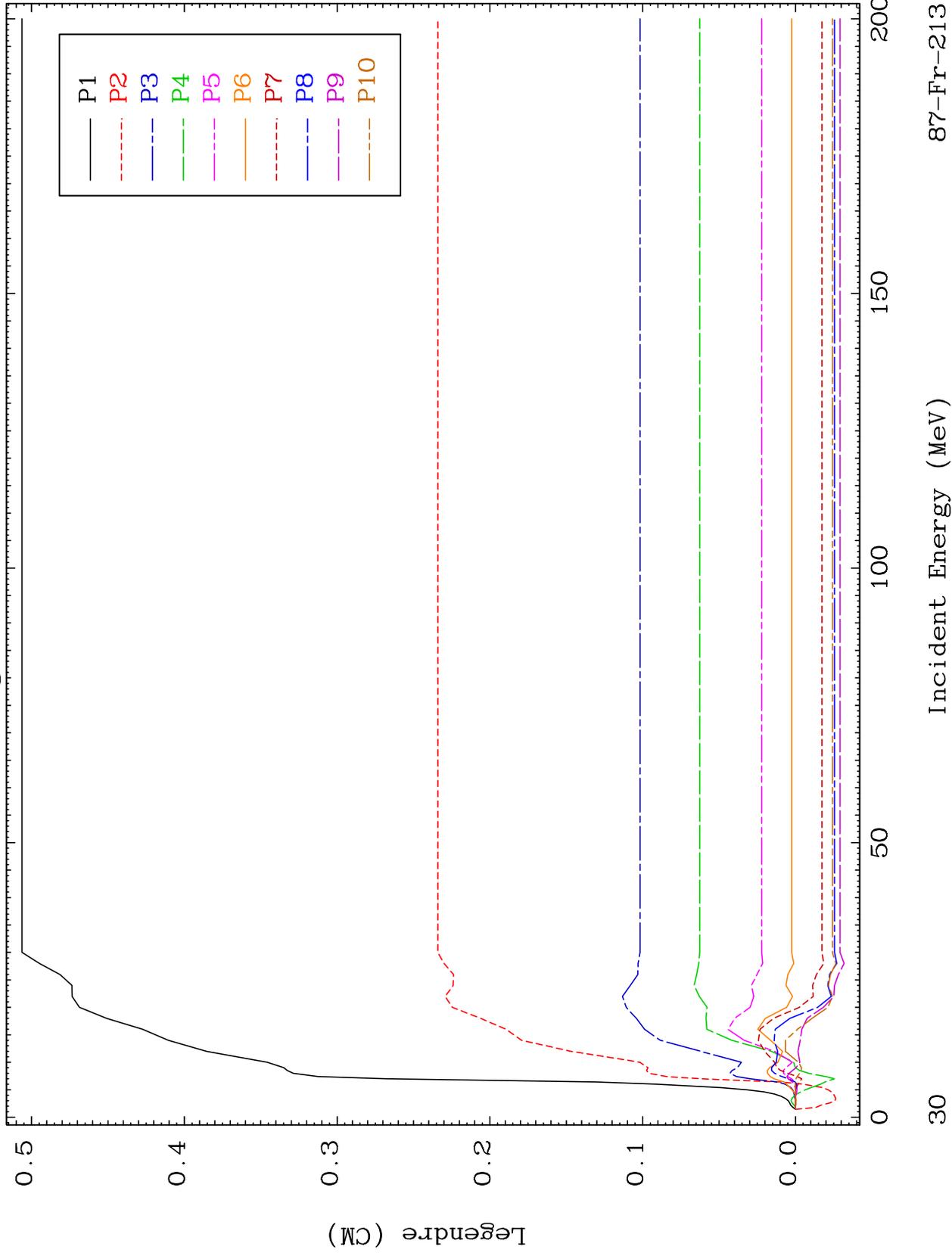
Incident Energy (MeV)

87-Fr-213

MAT 8728

MT= 54 (n,n') Level
Legendre Coefficients

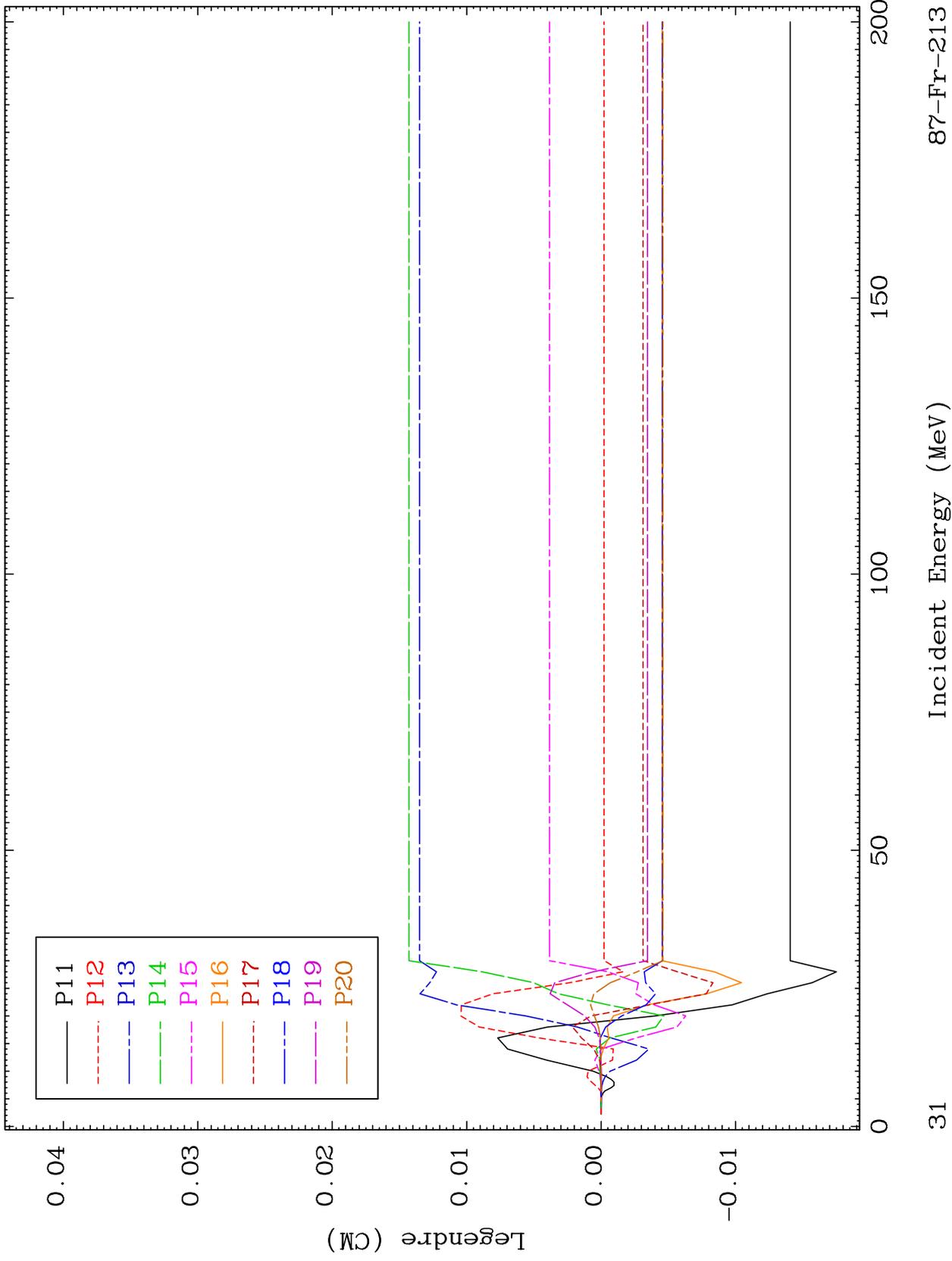
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MAT 8728

MT= 54 (n,n') Level
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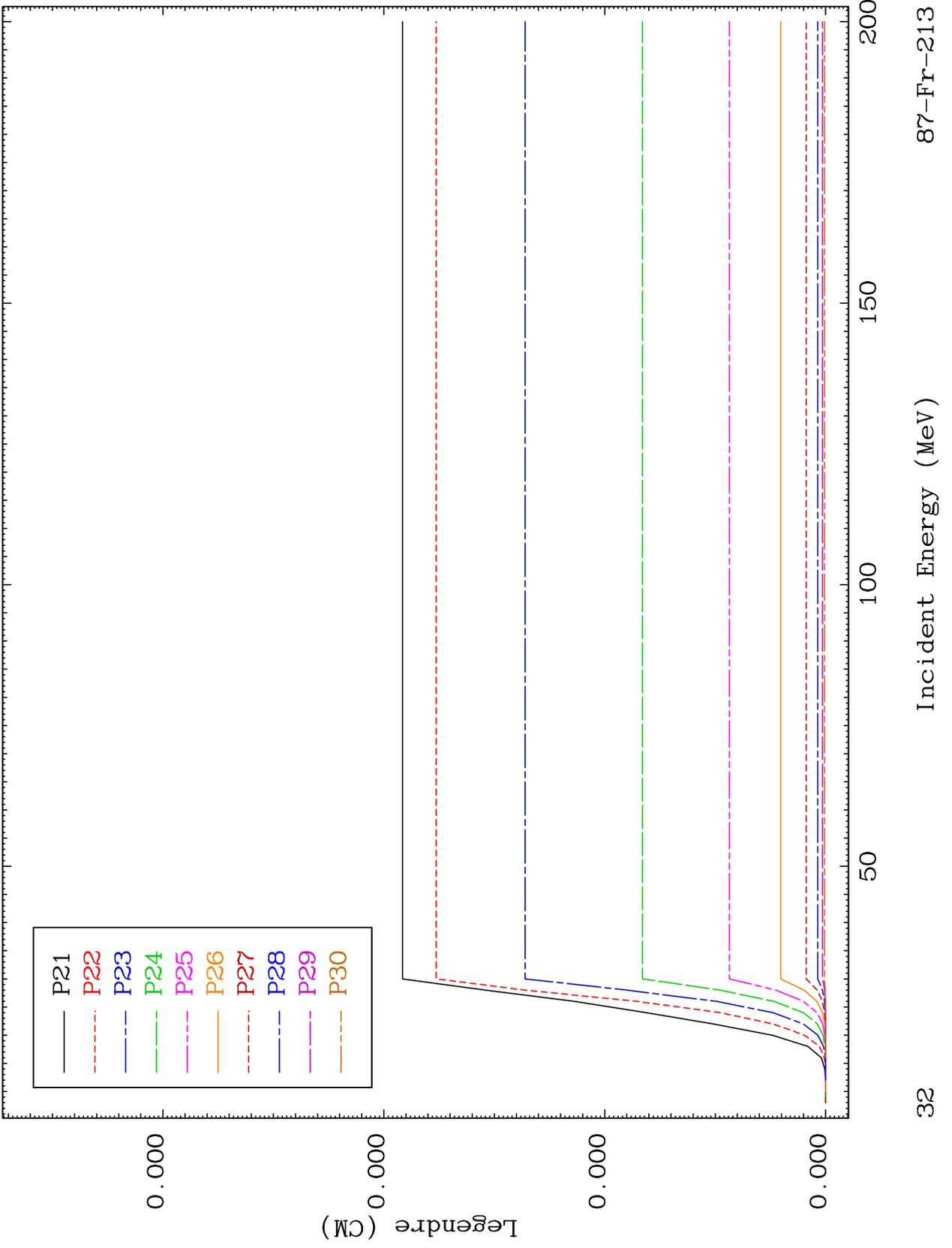
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MAT 8728

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Legendre Coefficients

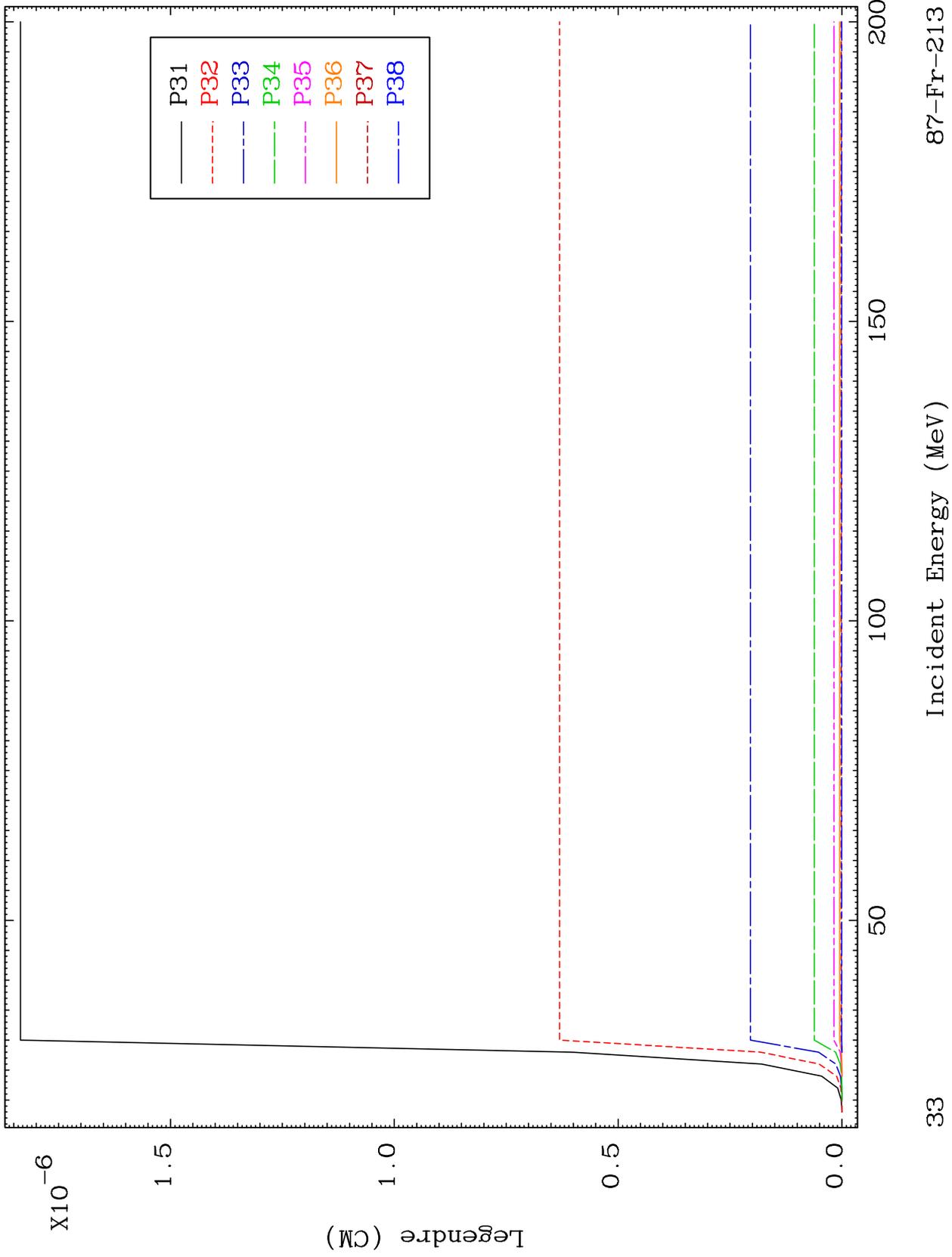
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MAT 8728

MT= 54 (n,n') Level
Legendre Coefficients

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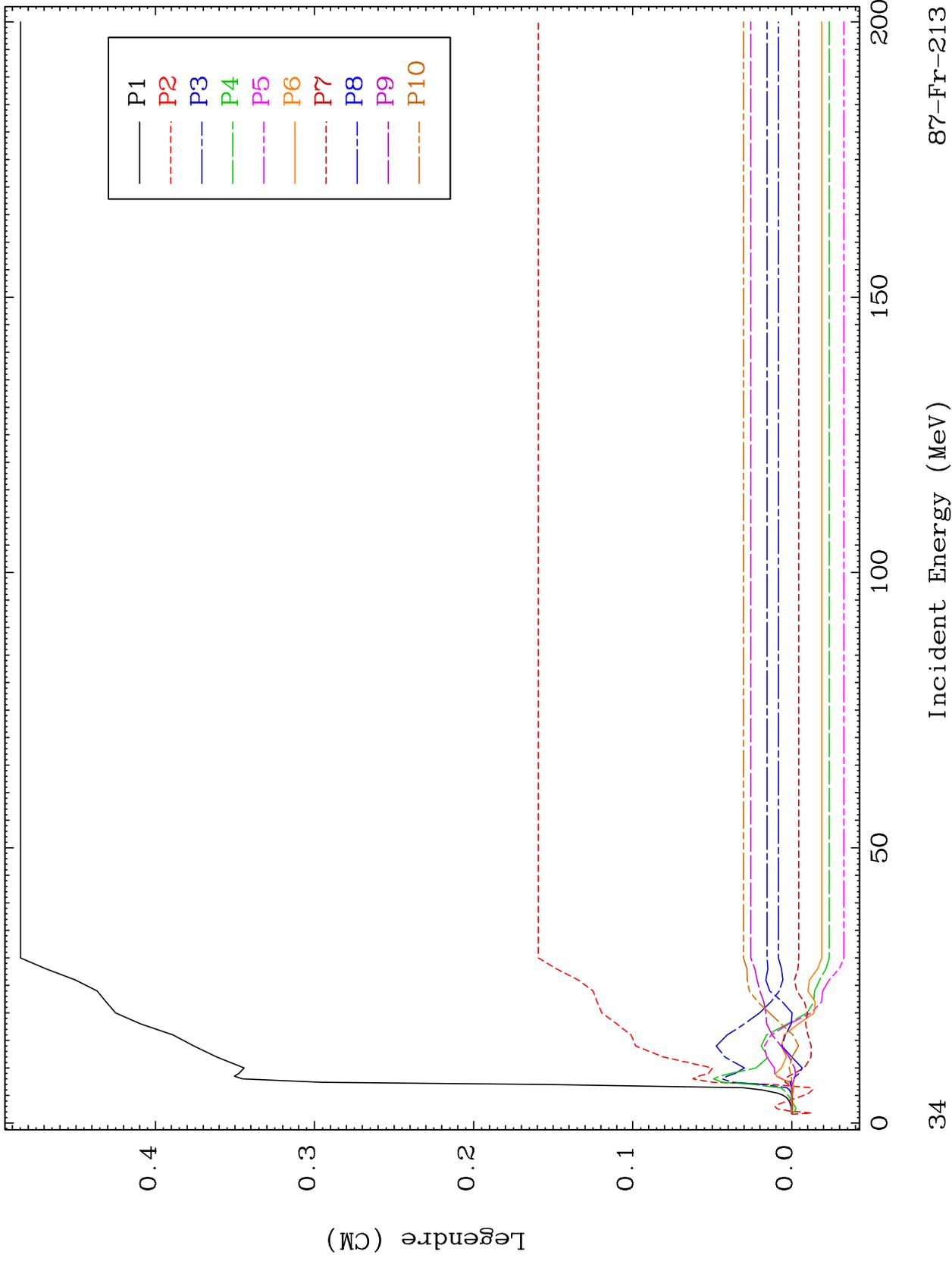


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MAT 8728

MT= 55 (n,n') Level
Legendre Coefficients

87-Fr-213



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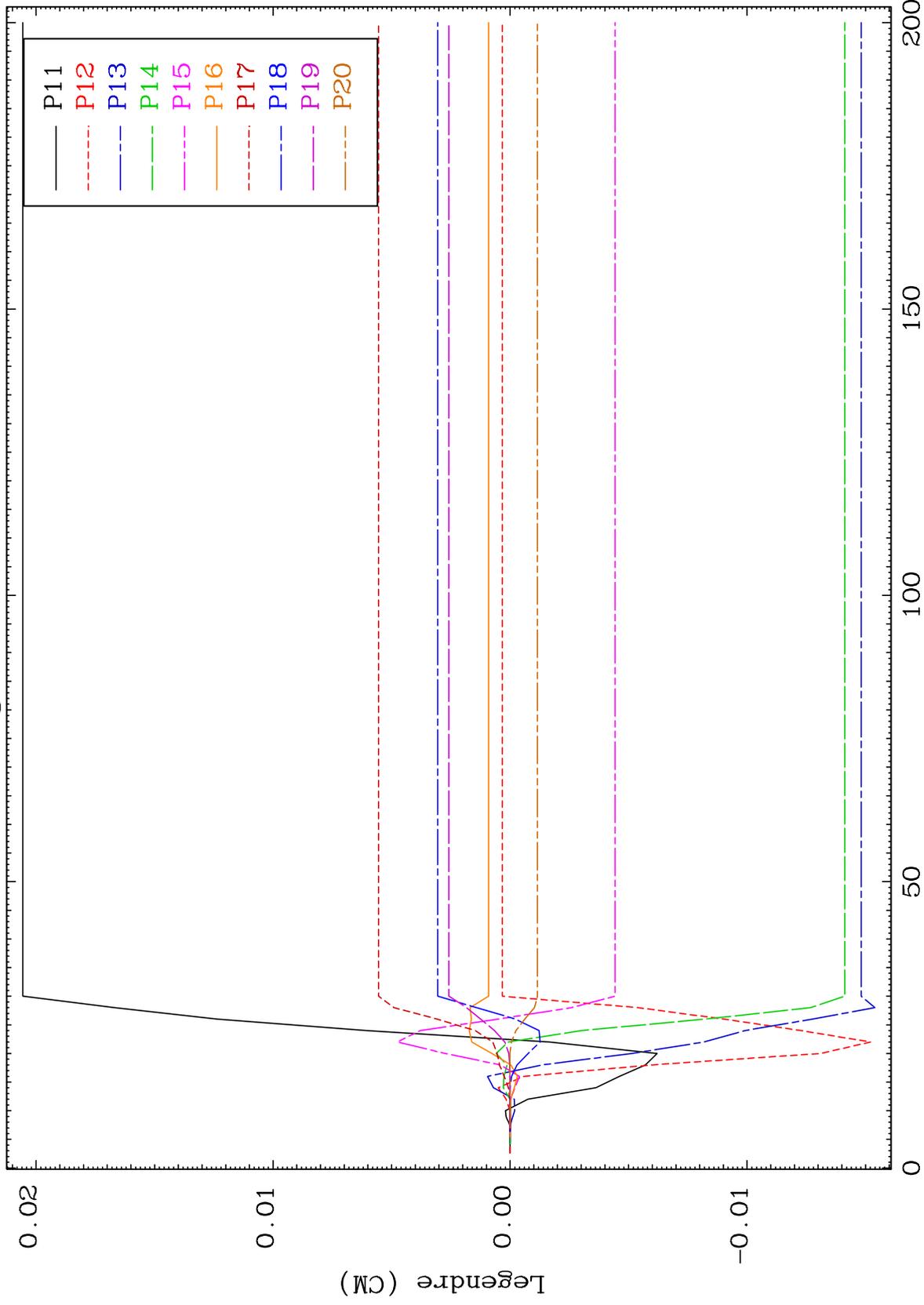
Incident Energy (MeV)

87-Fr-213

MAT 8728

MT= 55 (n,n') Level
Legendre Coefficients

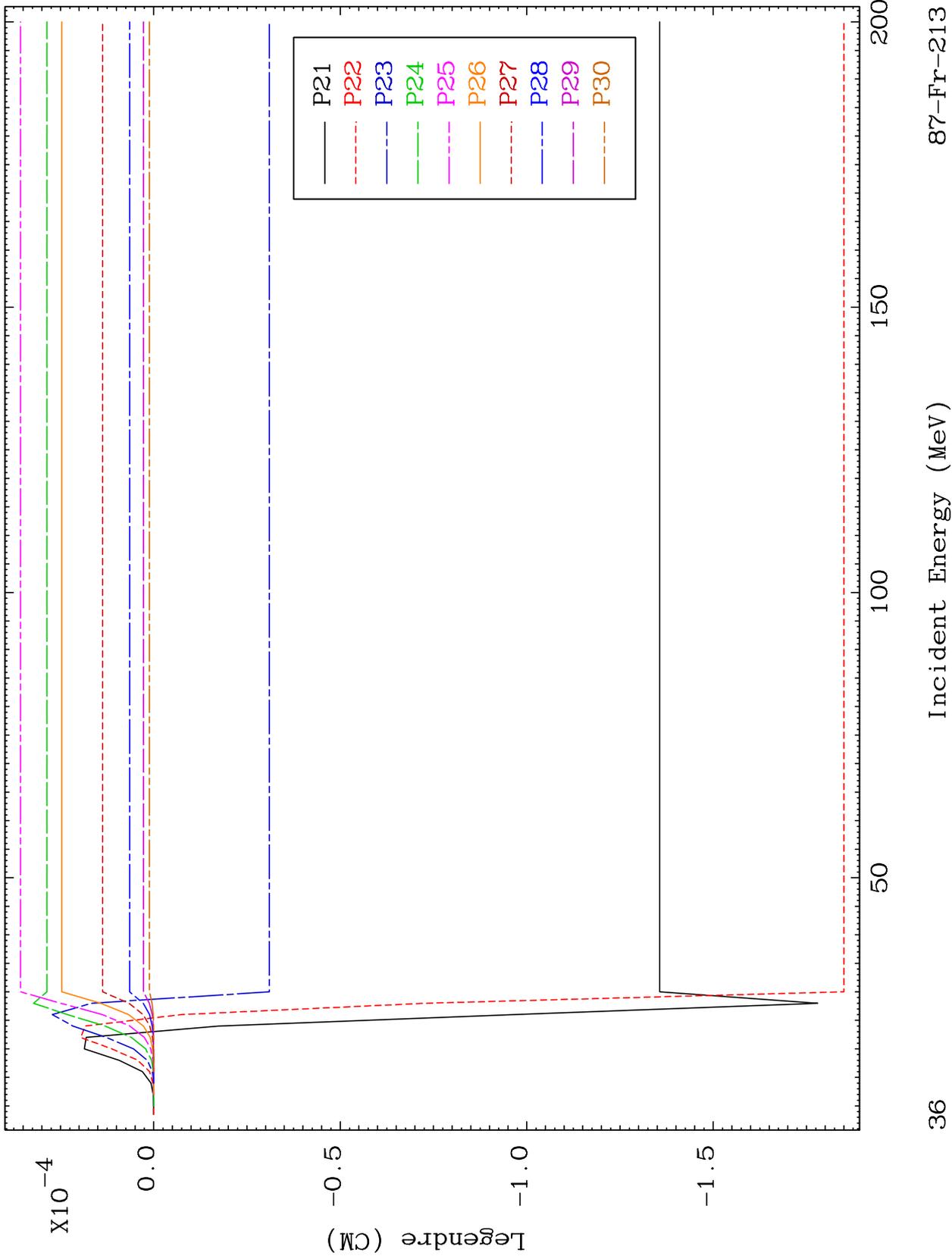
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35

Incident Energy (MeV)

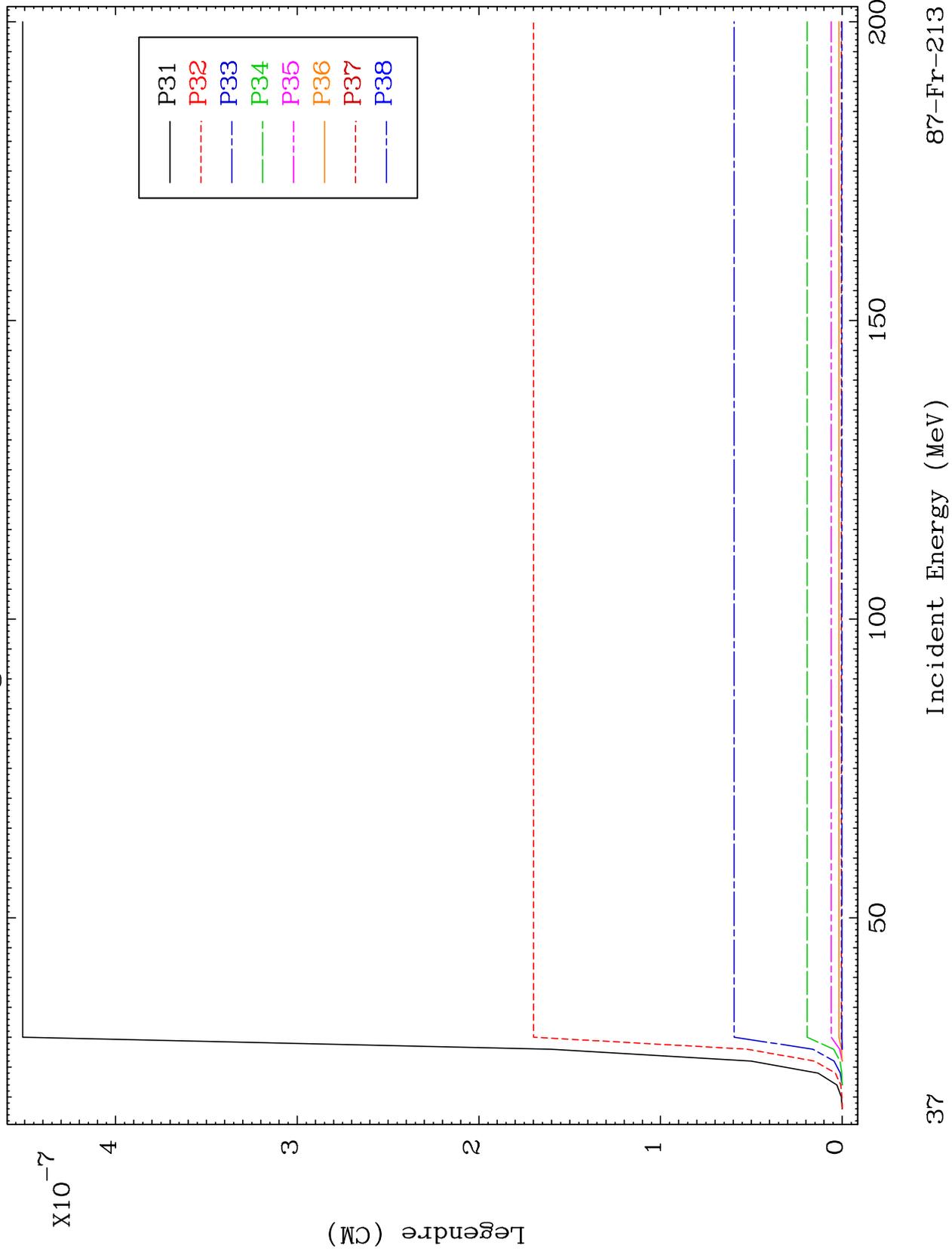
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MAT 8728

MT= 55 (n,n') Level
Legendre Coefficients

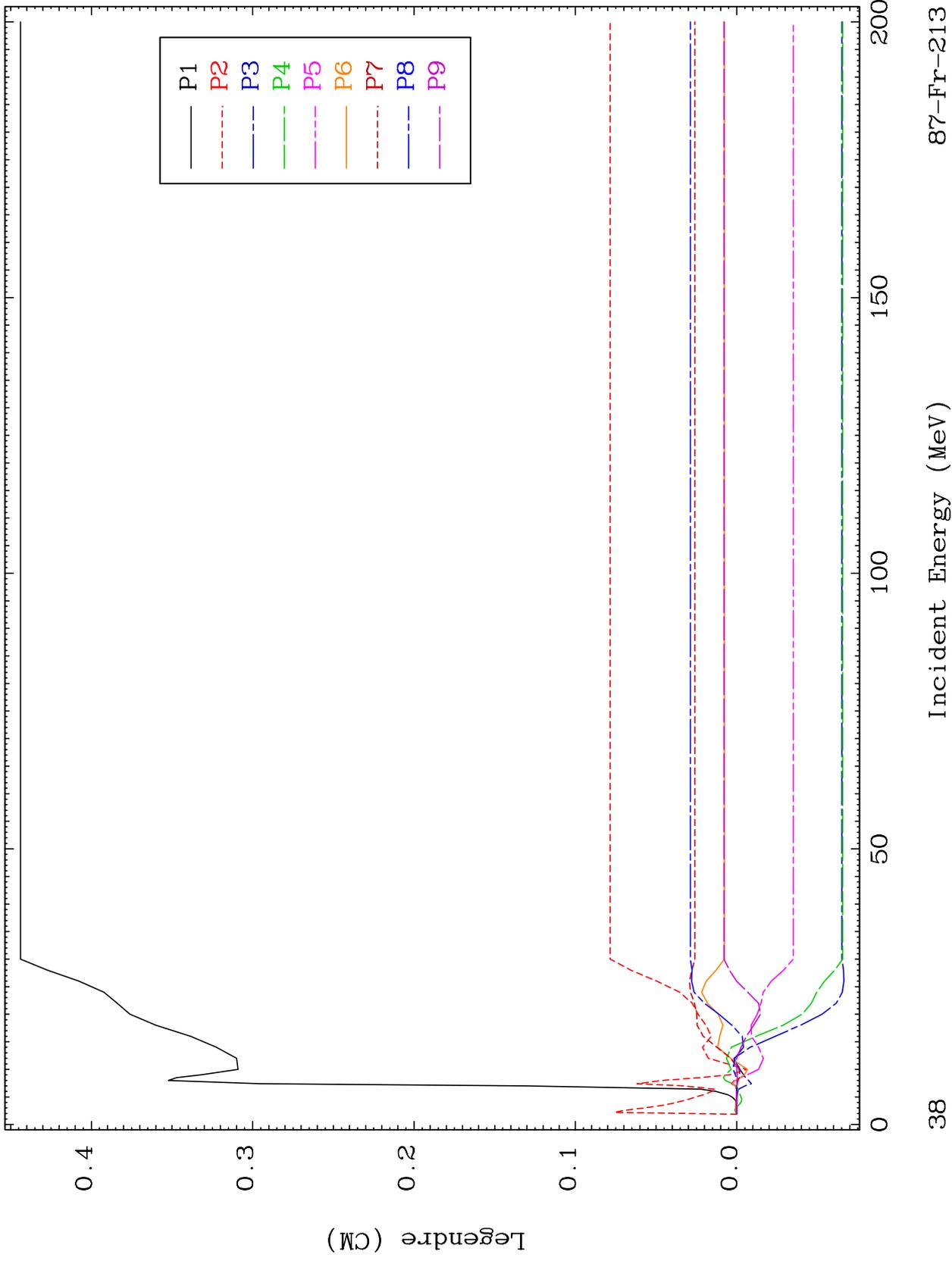
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MAT 8728

MT= 56 (n,n') Level
Legendre Coefficients

87-Fr-213



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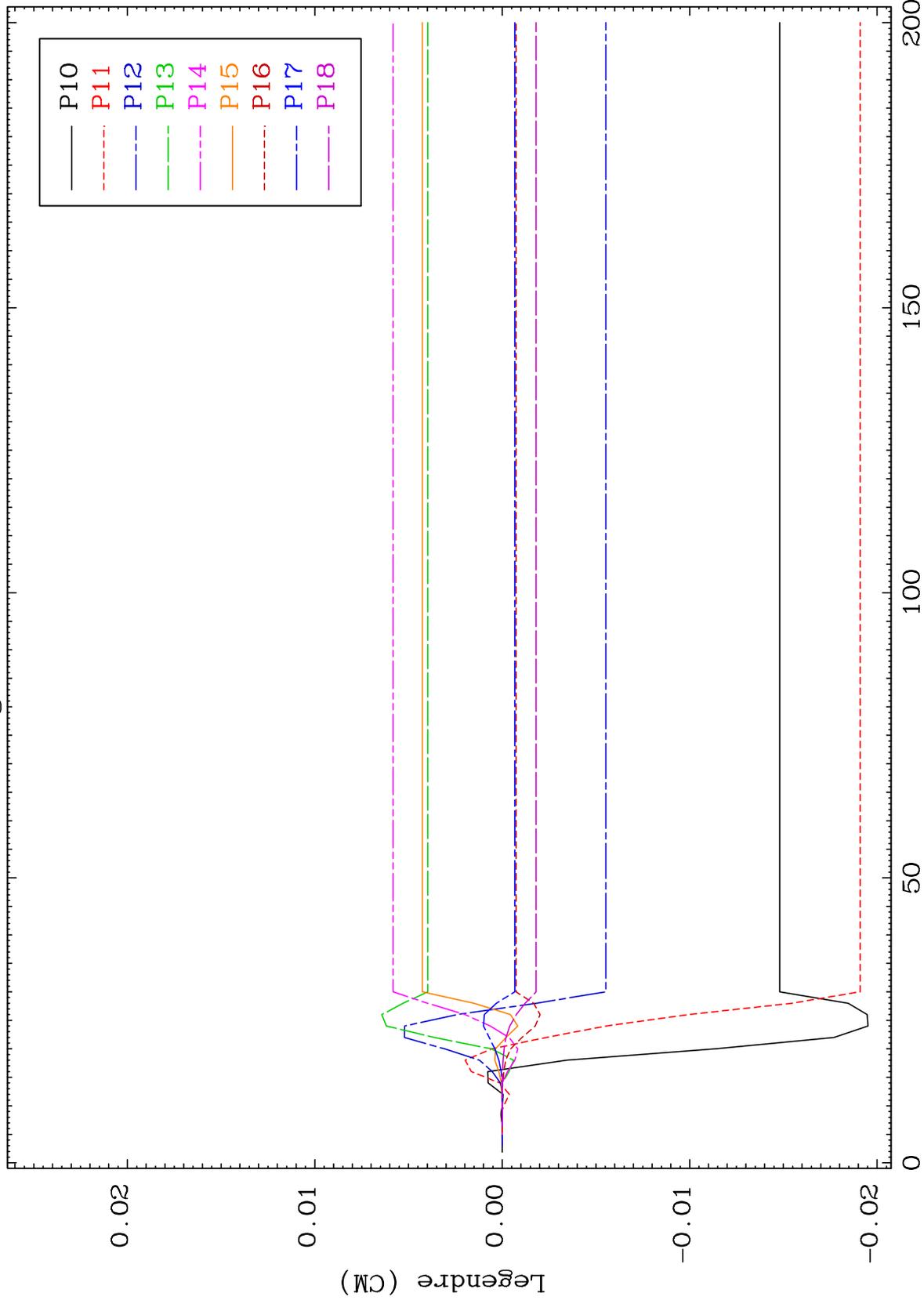
Incident Energy (MeV)

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MAT 8728

MT= 56 (n,n') Level
Legendre Coefficients

87-Fr-213



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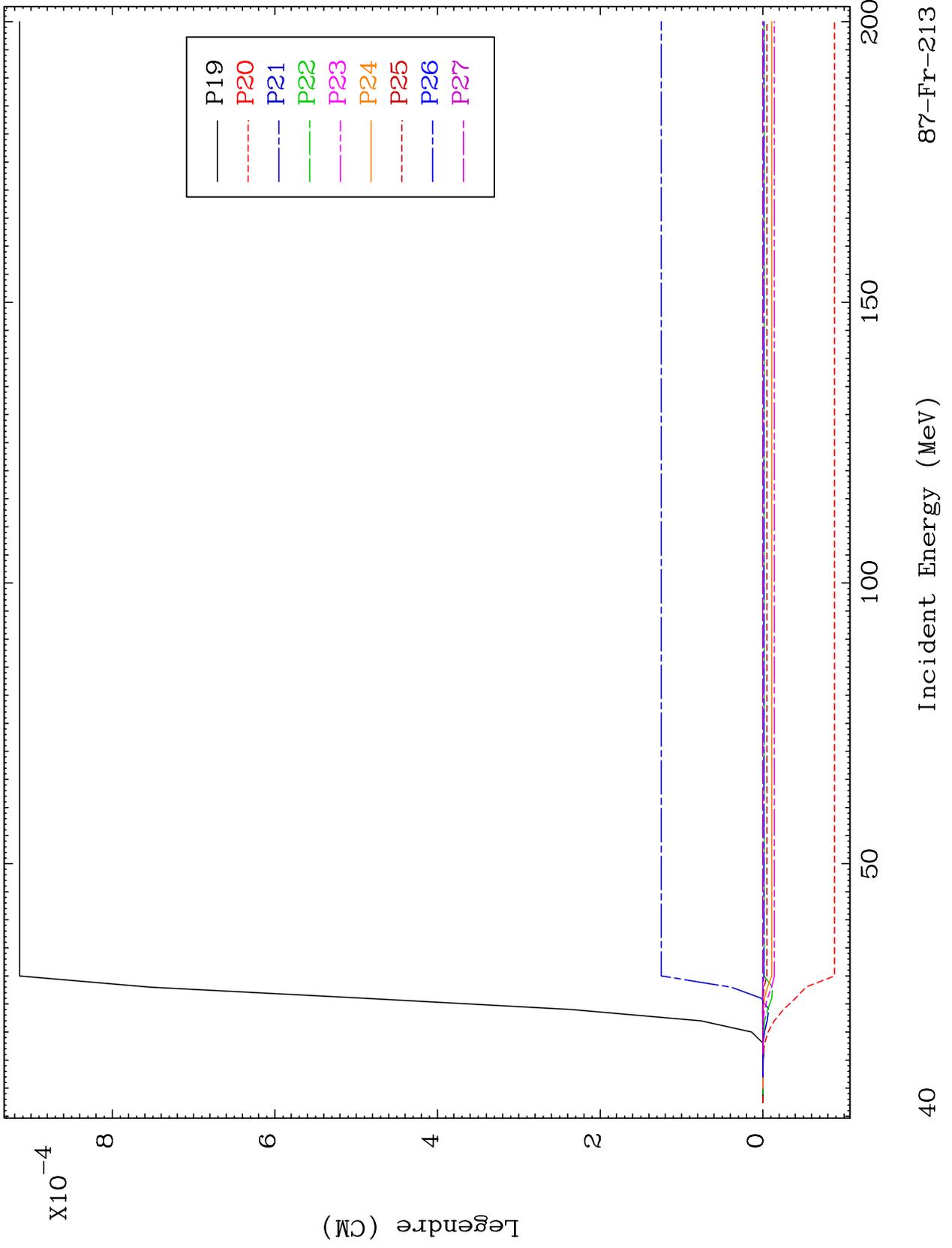
Incident Energy (MeV)

87-Fr-213

MAT 8728

MT= 56 (n,n') Level
Legendre Coefficients

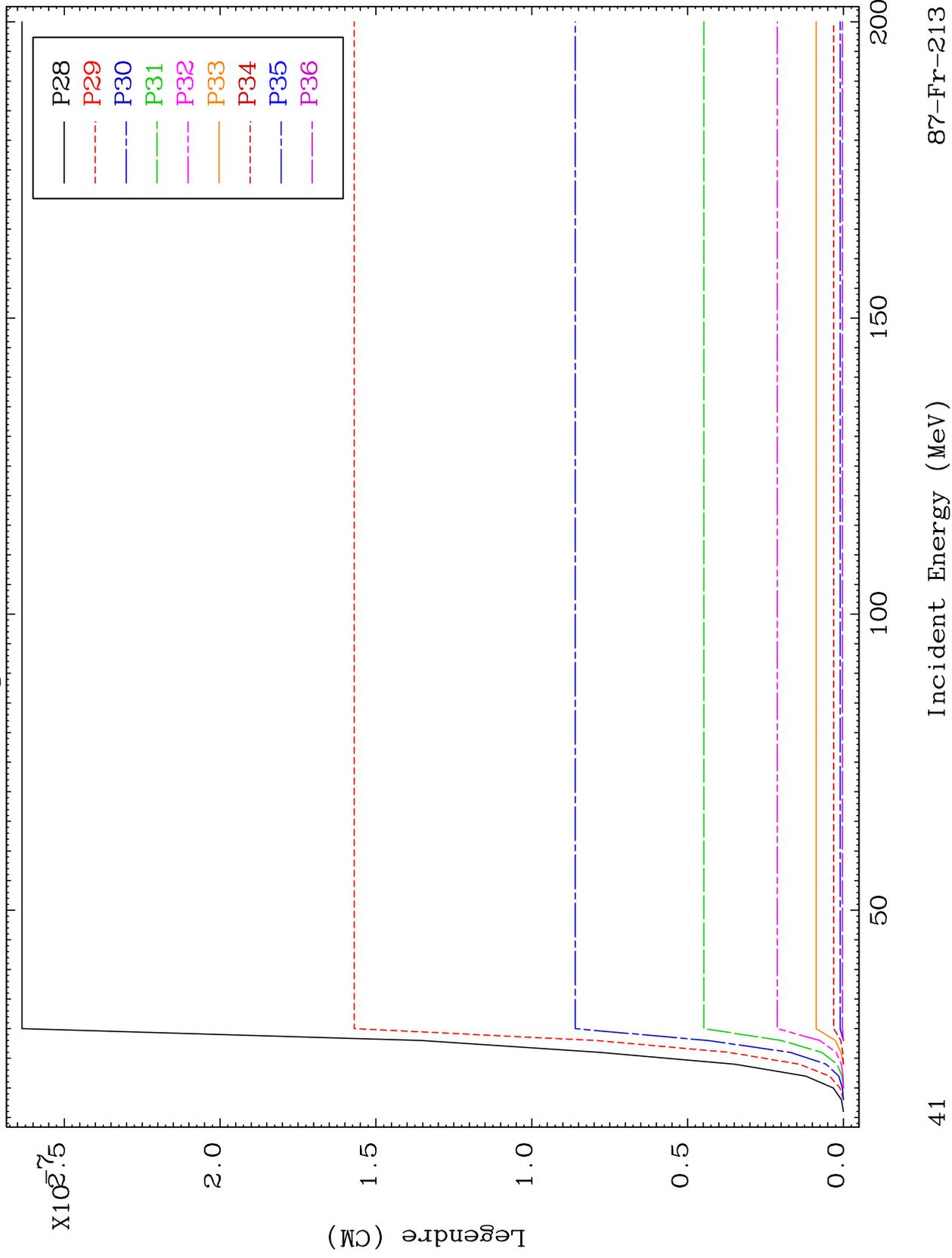
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MAT 8728

MT= 56 (n,n') Level
Legendre Coefficients

87-Fr-213

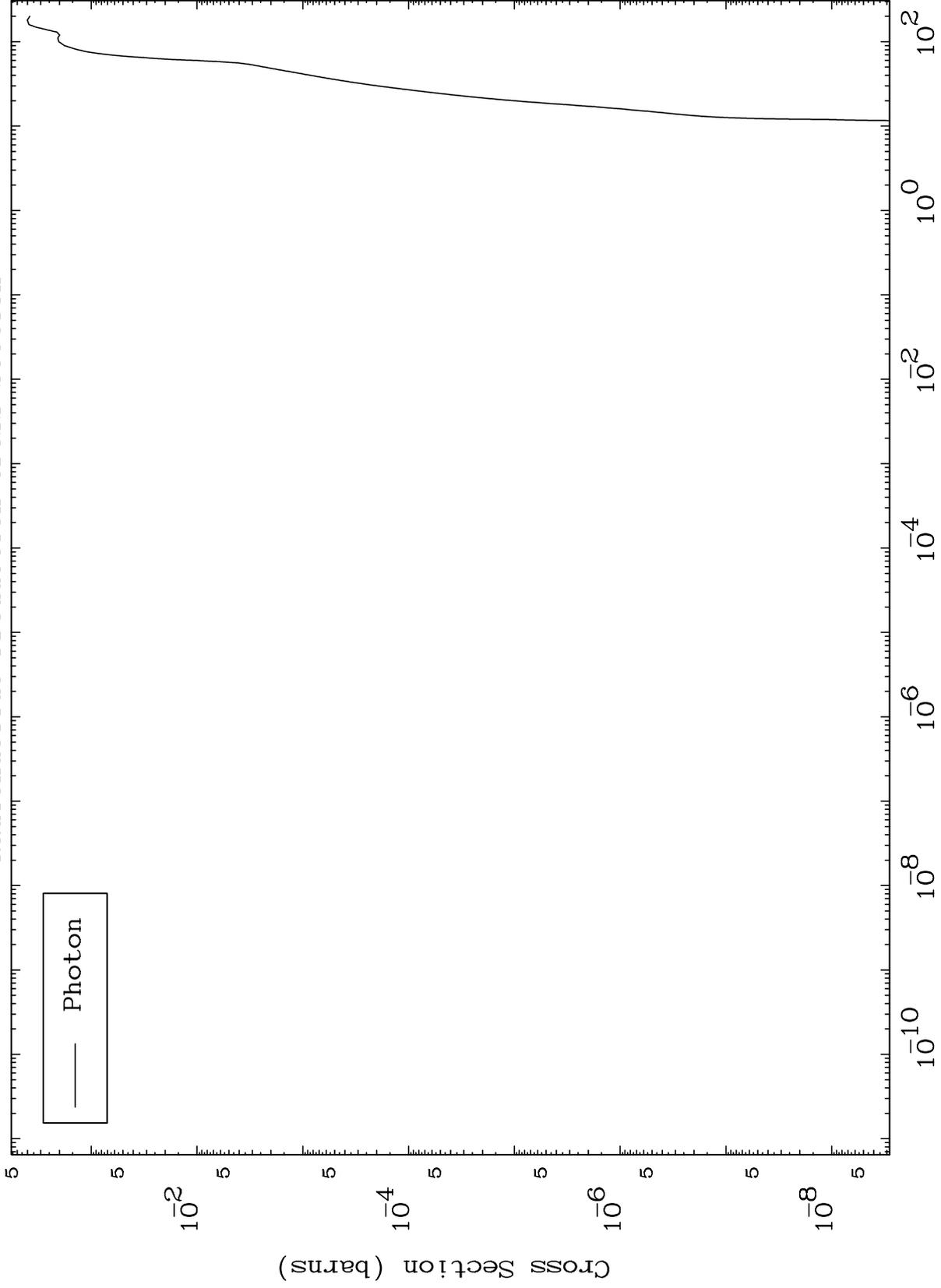


MAT 8728

Fission

87-Fr-213

Radionuclide Production Cross Section



42

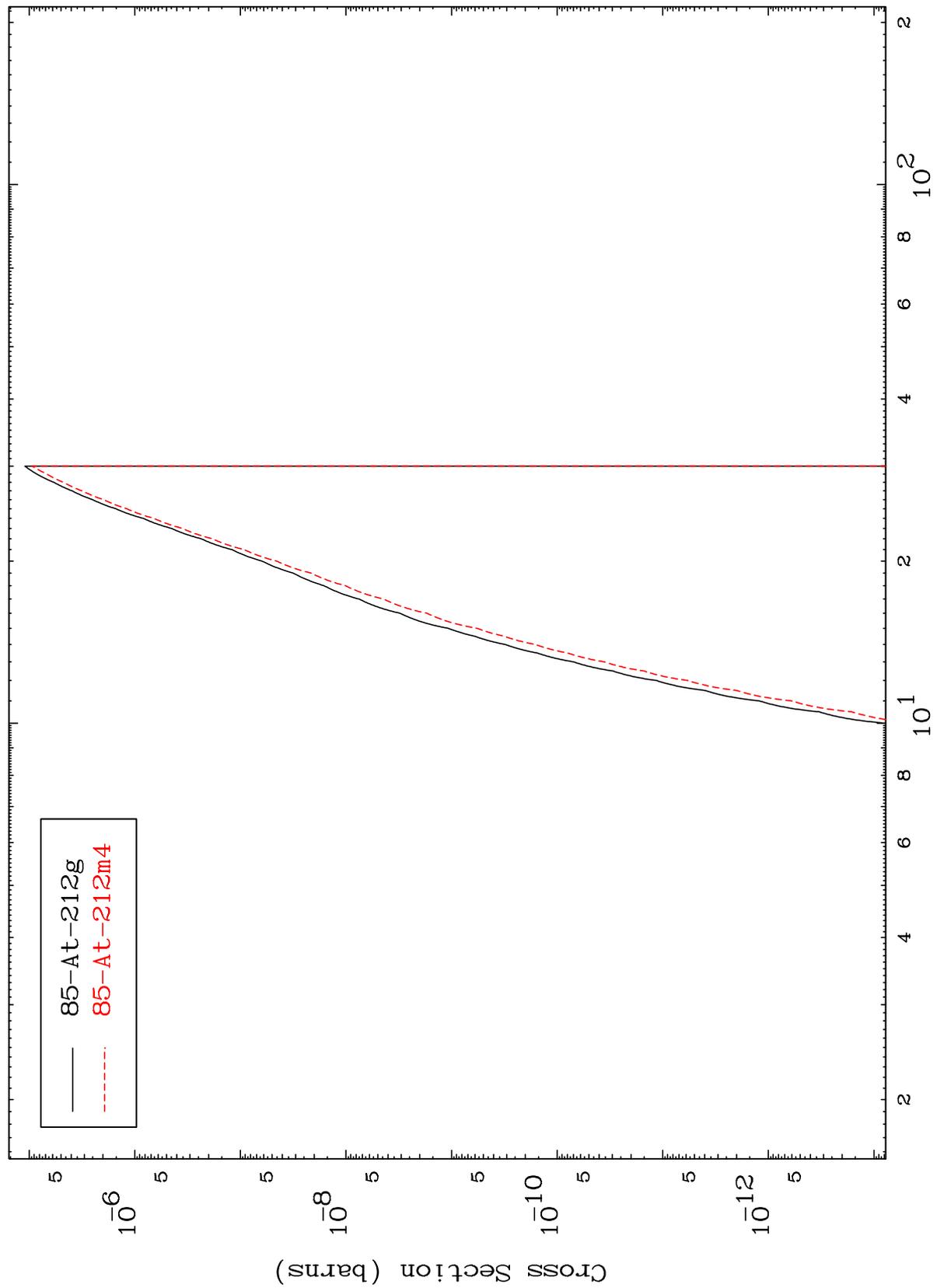
Incident Energy (MeV)

87-Fr-213

MAT 8728

87-Fr-213

(n,2p)
Radionuclide Production Cross Section



43

87-Fr-213

Incident Energy (MeV)