

Program EVALPLOT  
(Version 2017-1)

by

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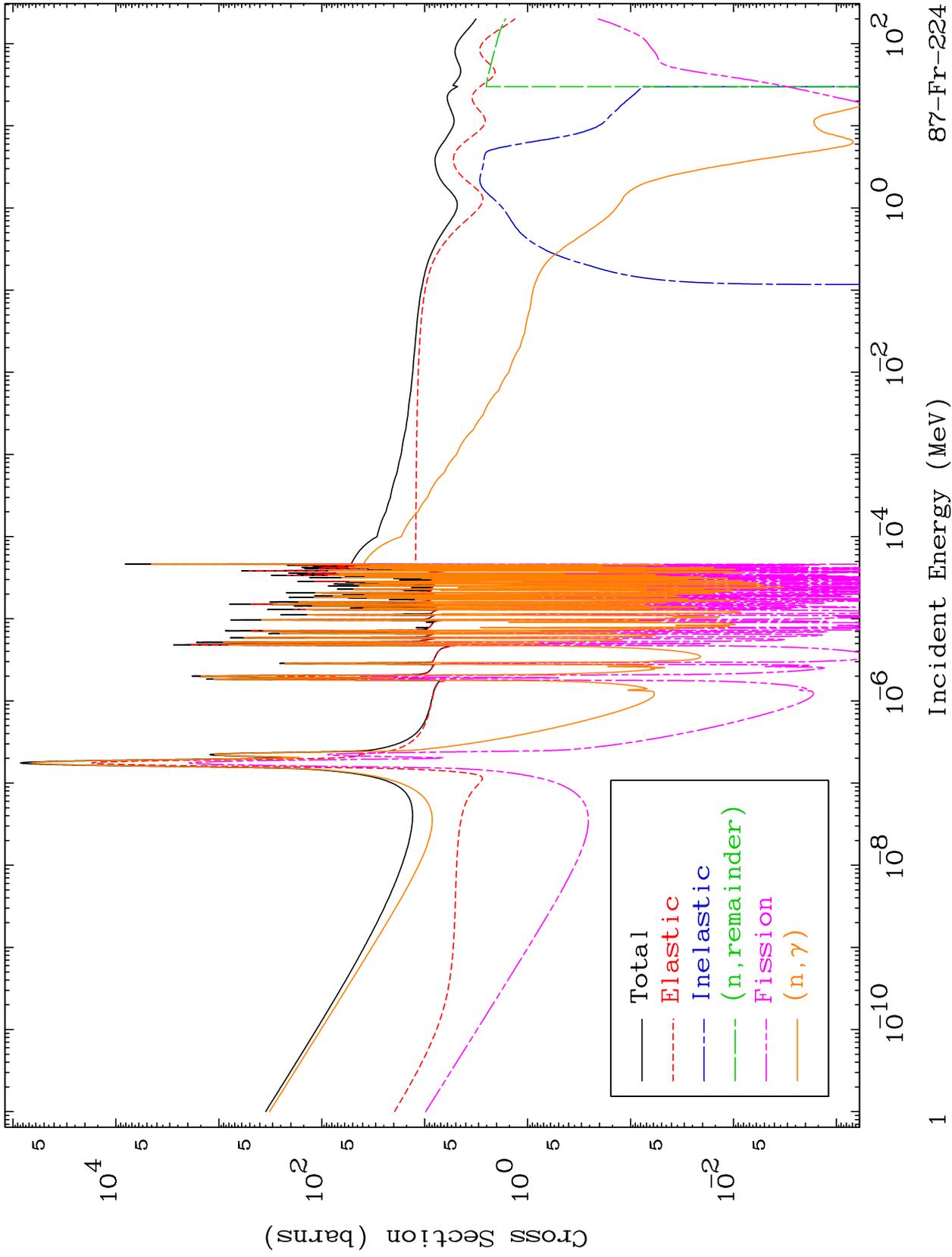
Press Mouse Button to Start

MAT 8761

Major

293 Kelvin Cross Sections

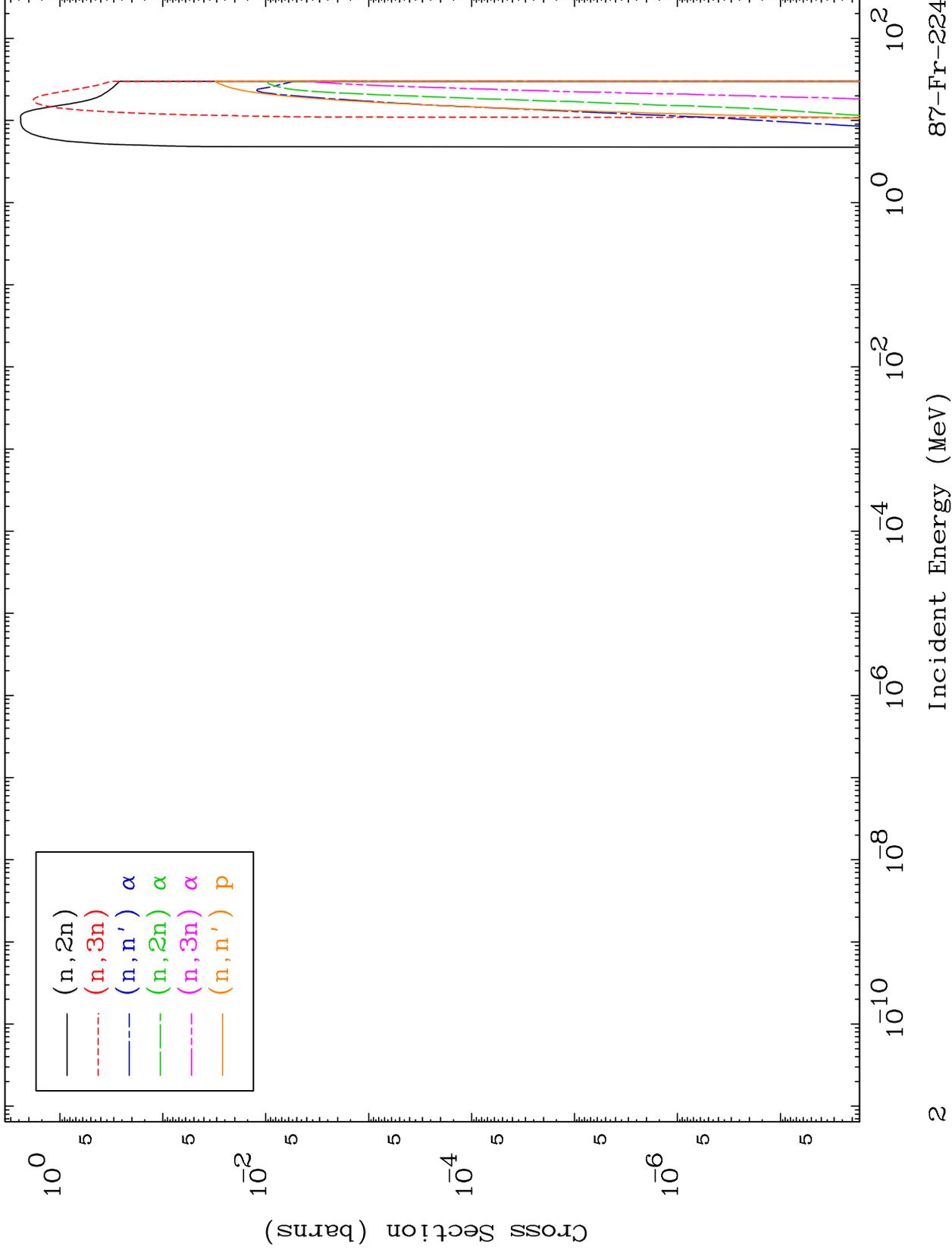
87-Fr-224



MAT 8761

Neutron Production  
293 Kelvin Cross Sections

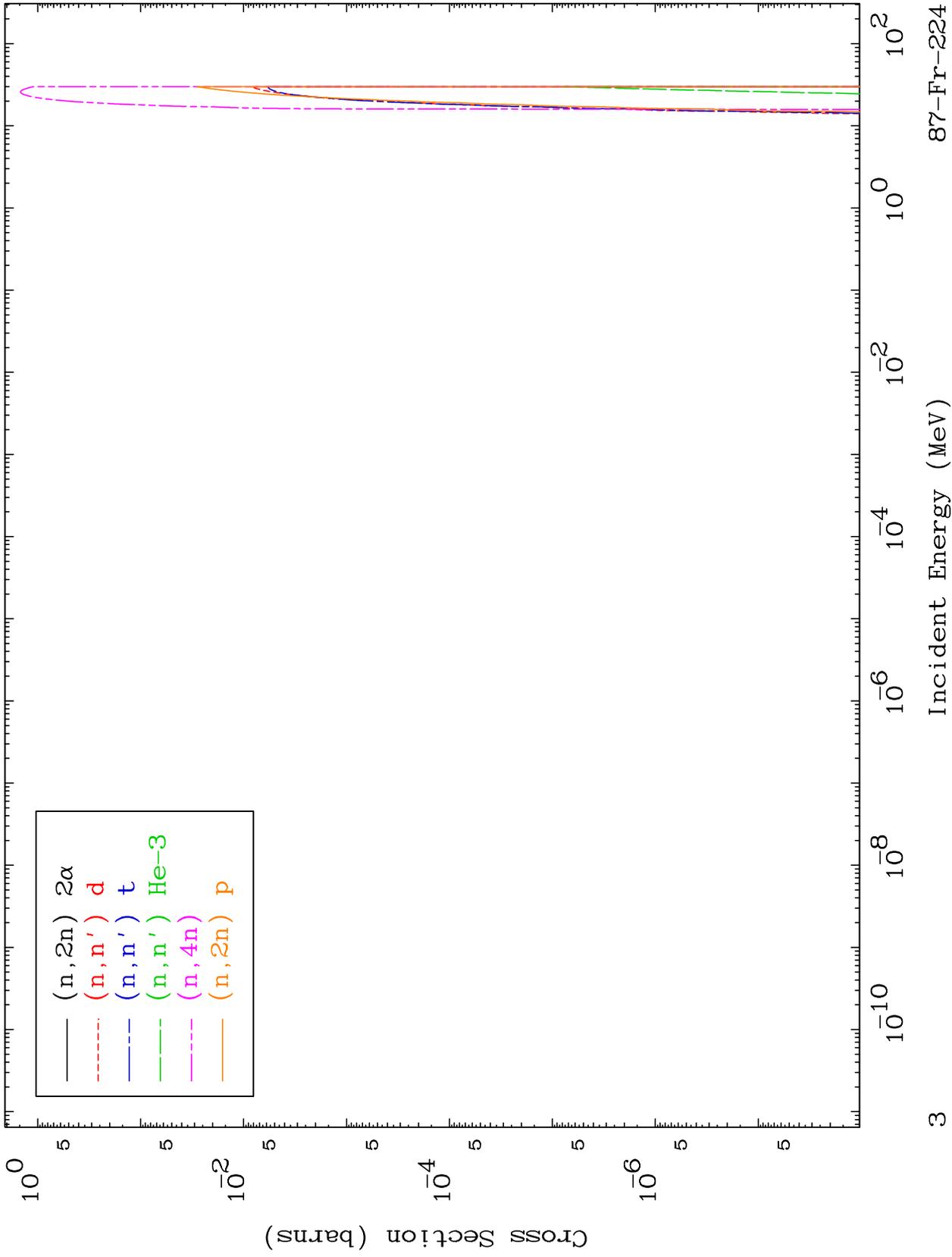
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Neutron Production  
293 Kelvin Cross Sections

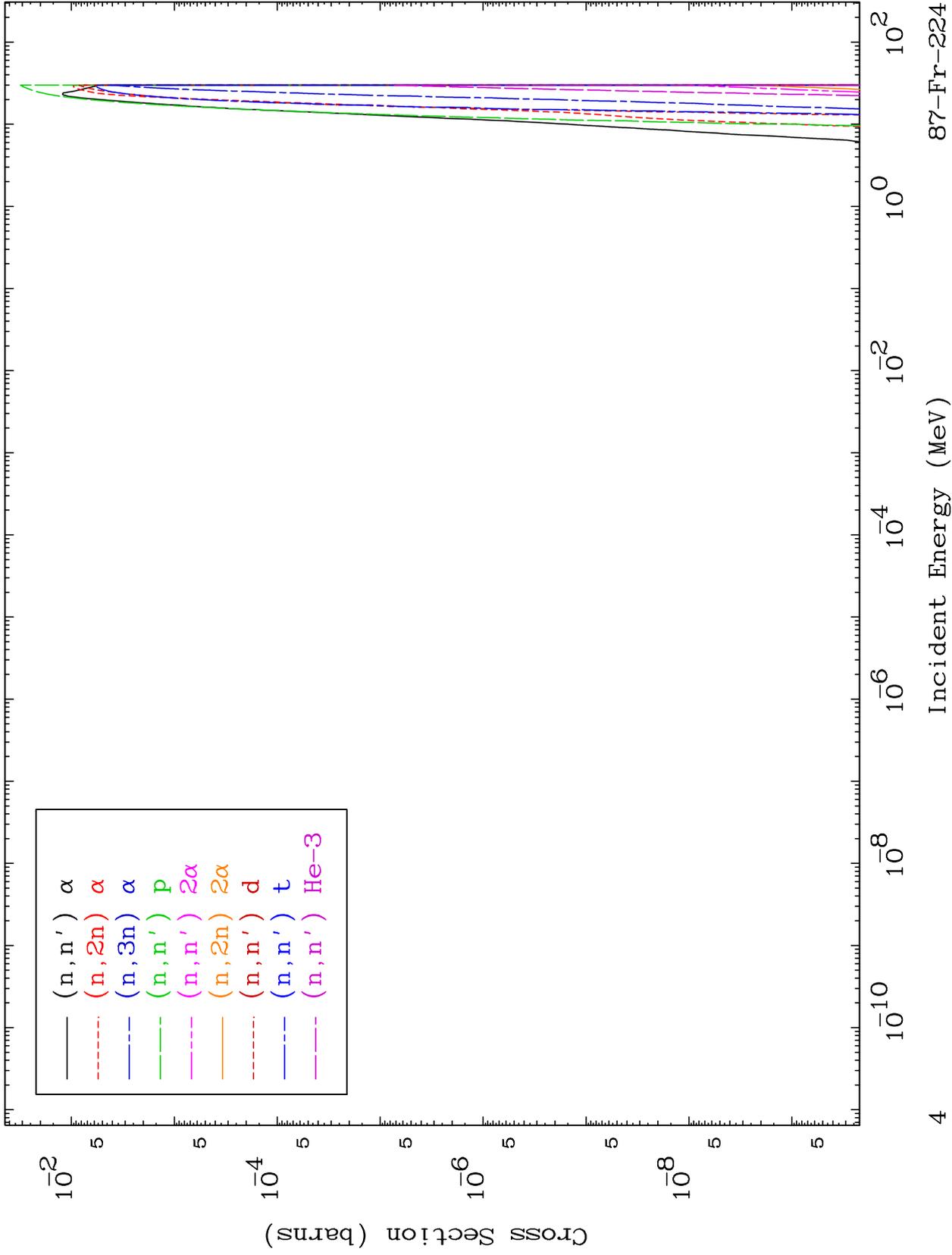
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Charged Particle  
293 Kelvin Cross Sections

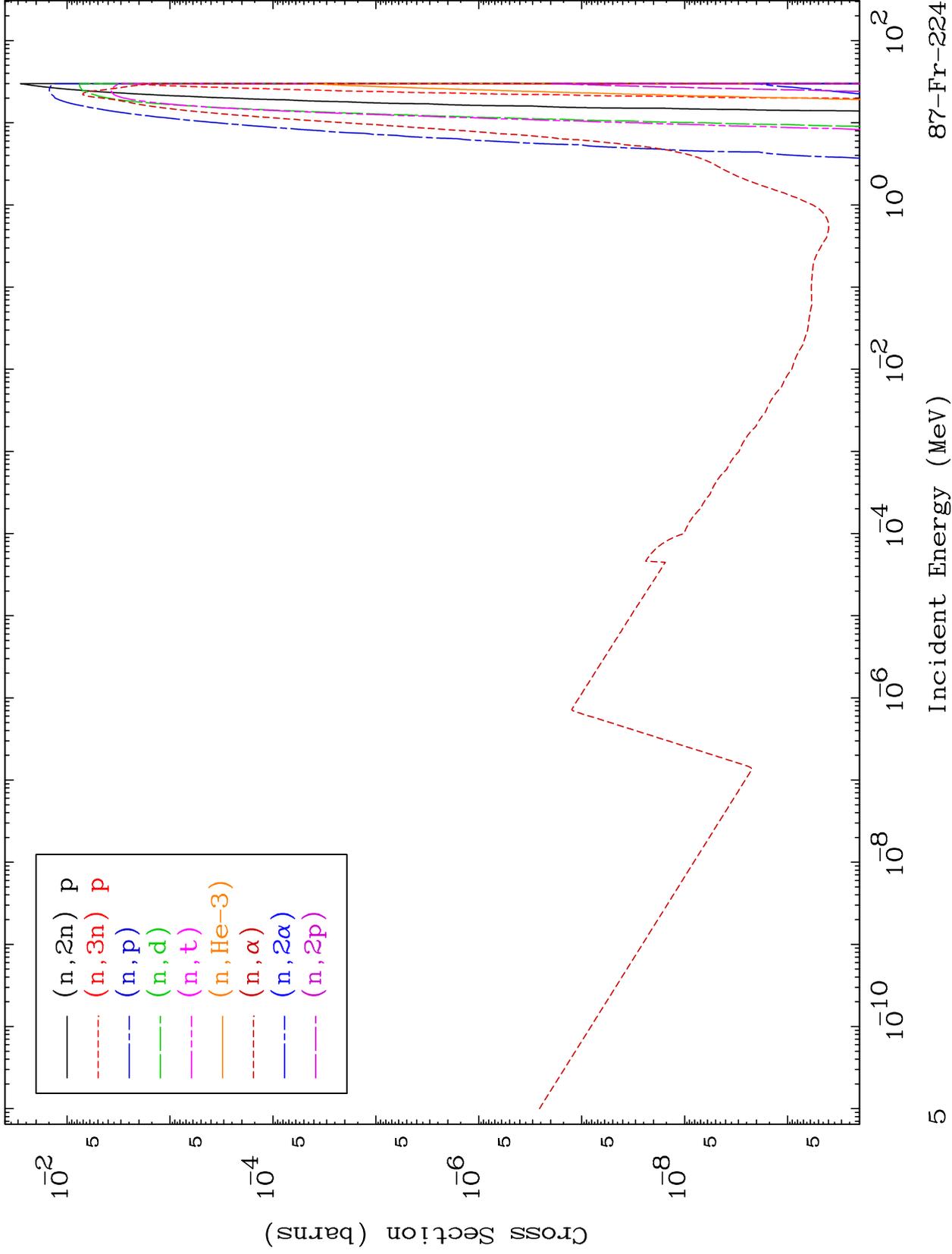
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Charged Particle  
293 Kelvin Cross Sections

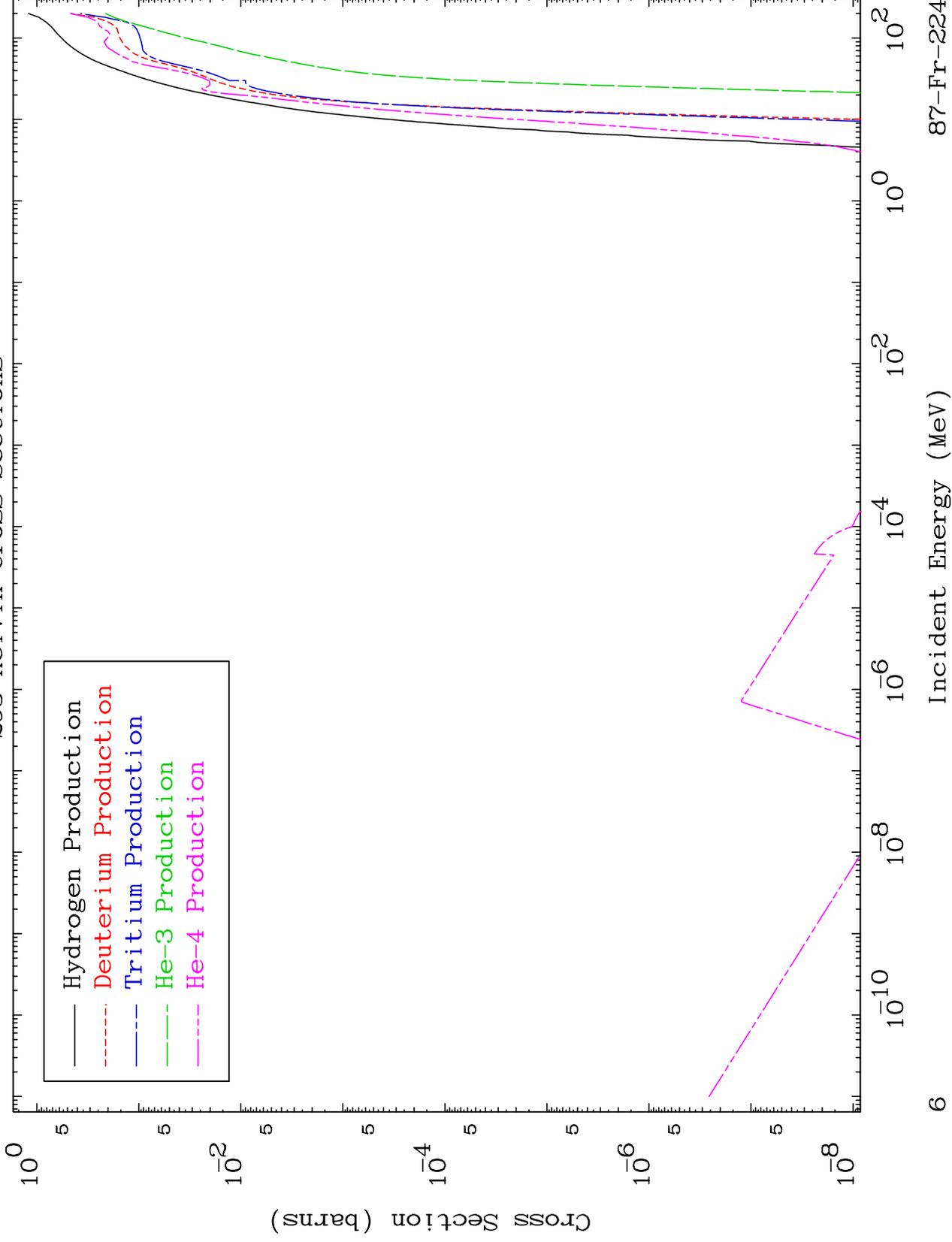
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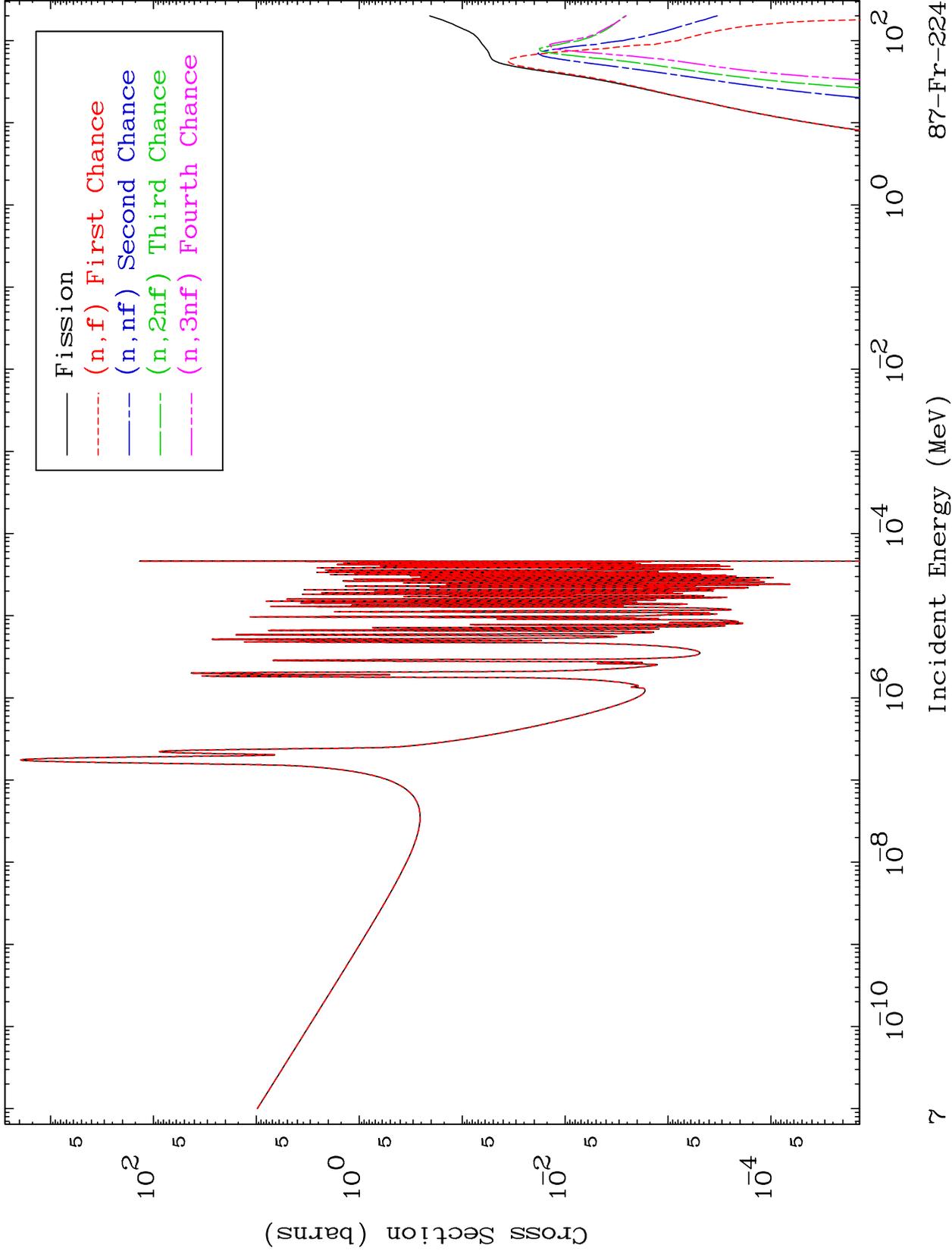


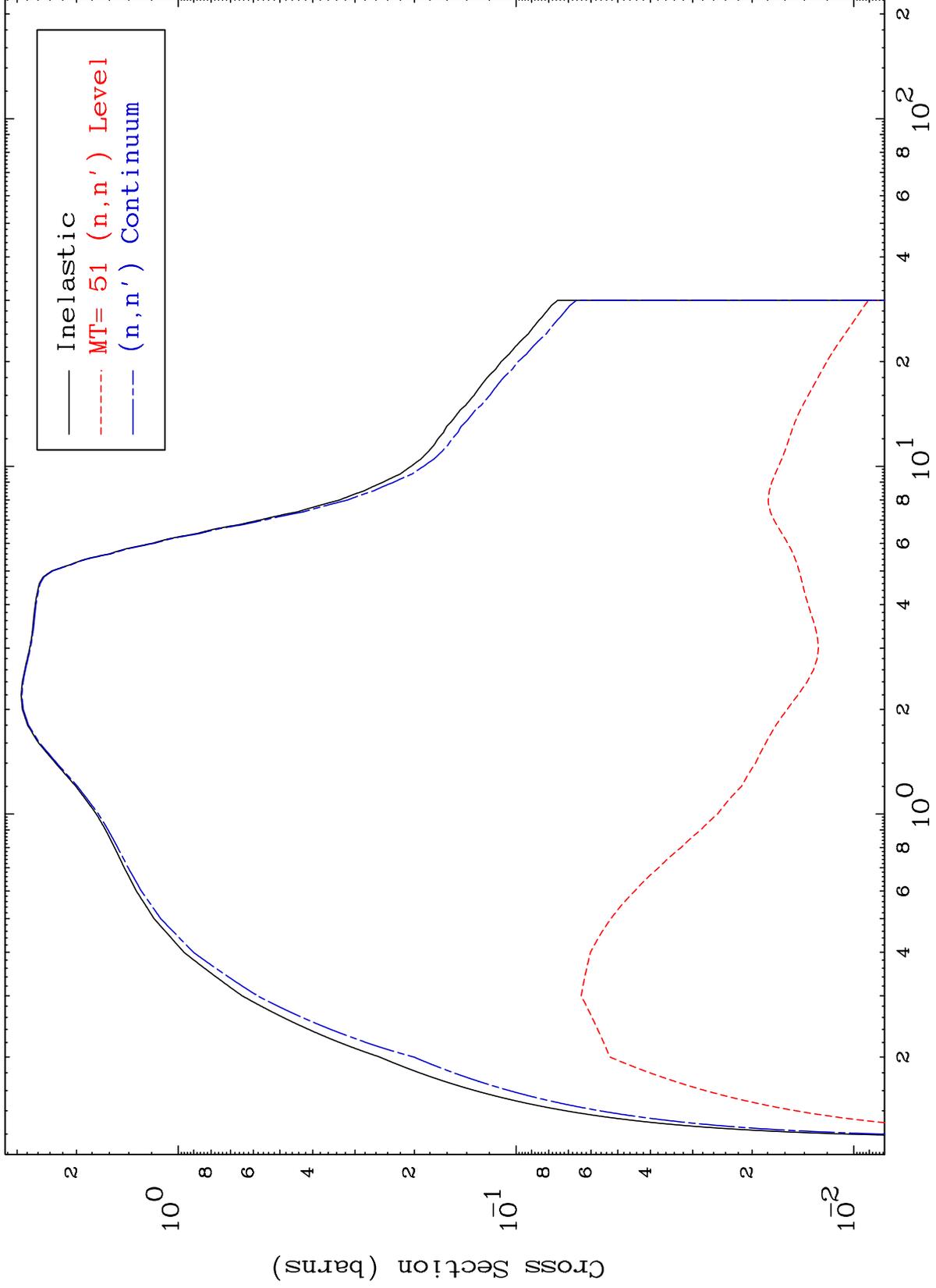
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Particle Production  
293 Kelvin Cross Sections

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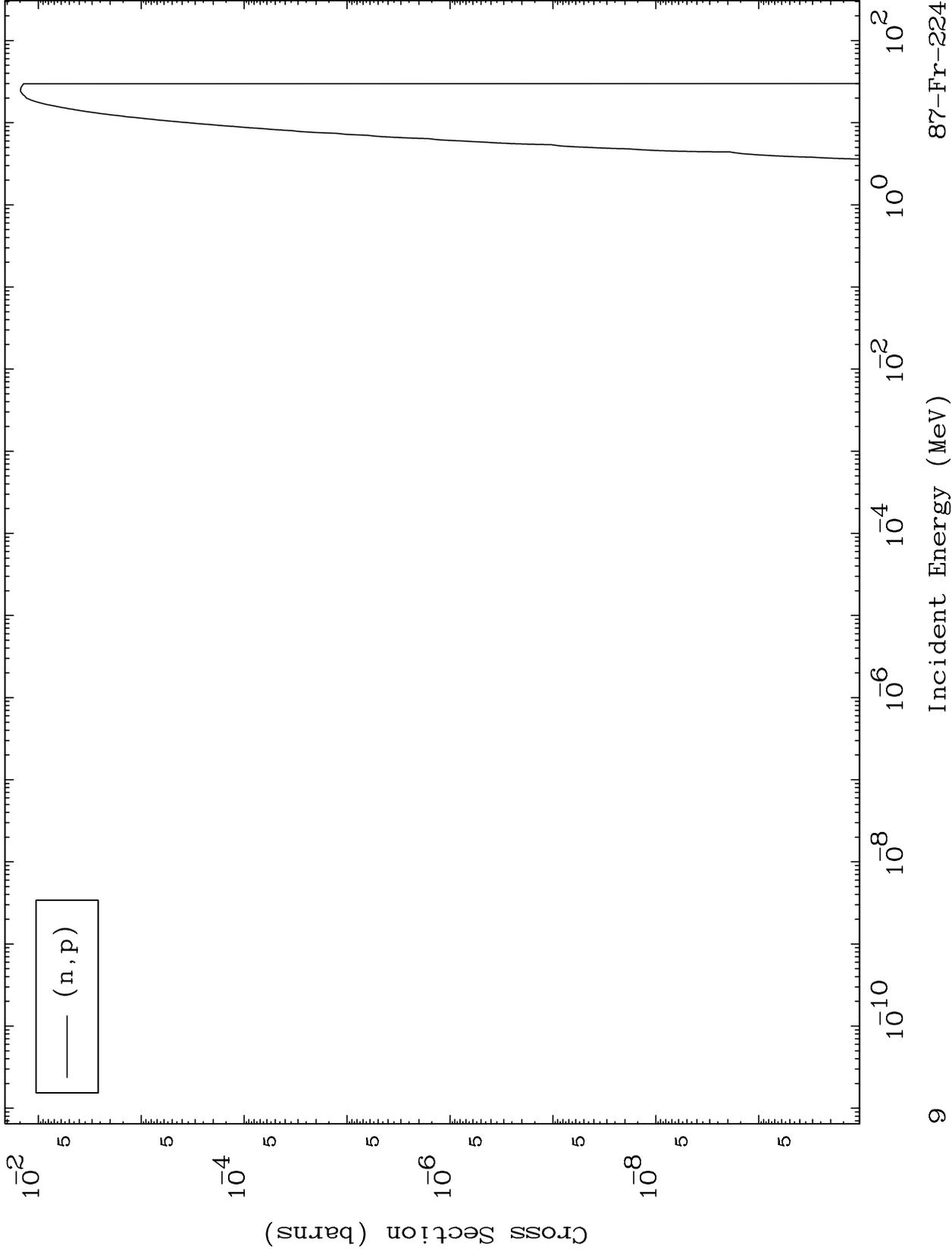




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(n,p) Levels  
293 Kelvin Cross Sections

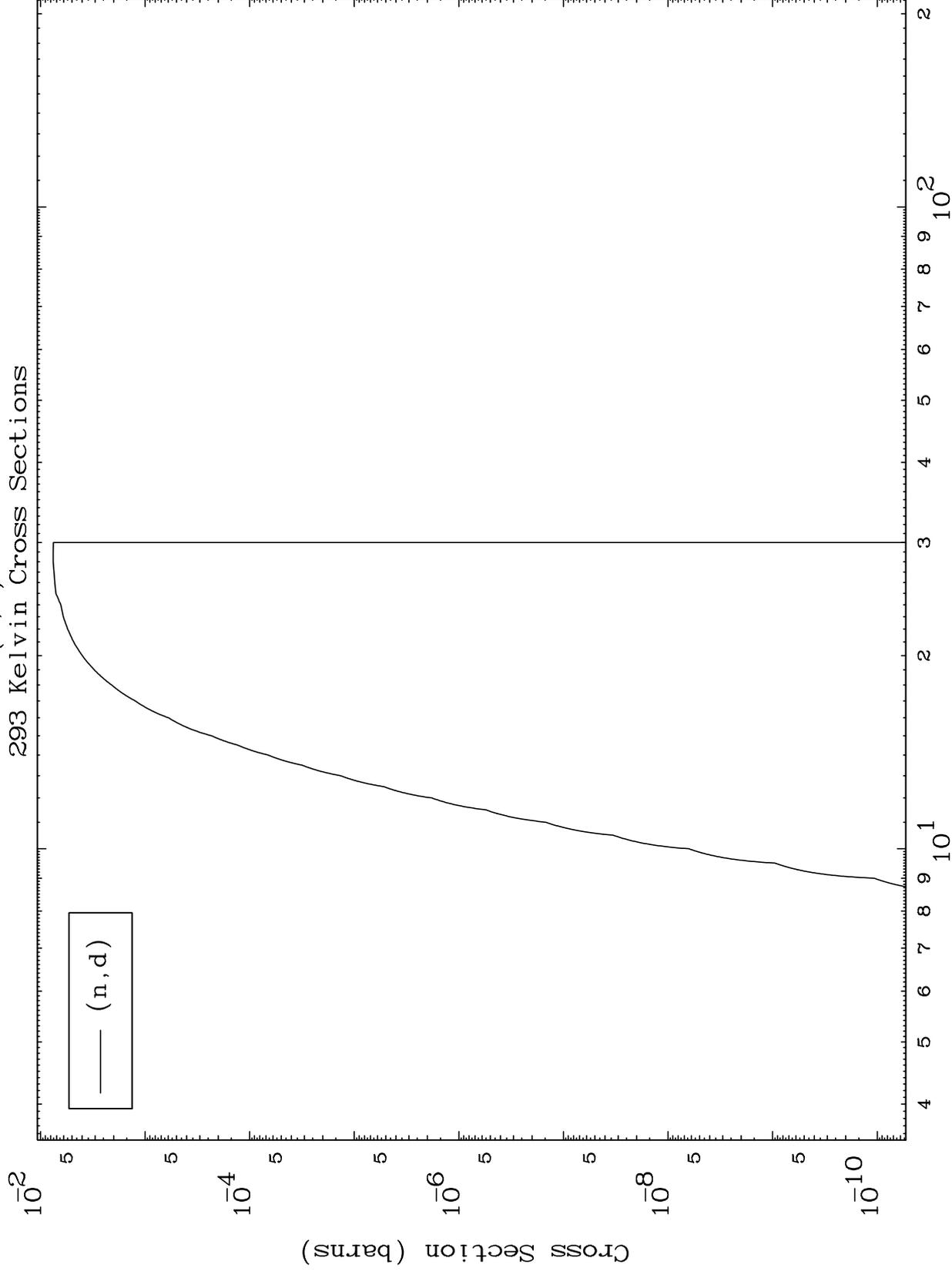
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(n,d) Levels  
293 Kelvin Cross Sections

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10

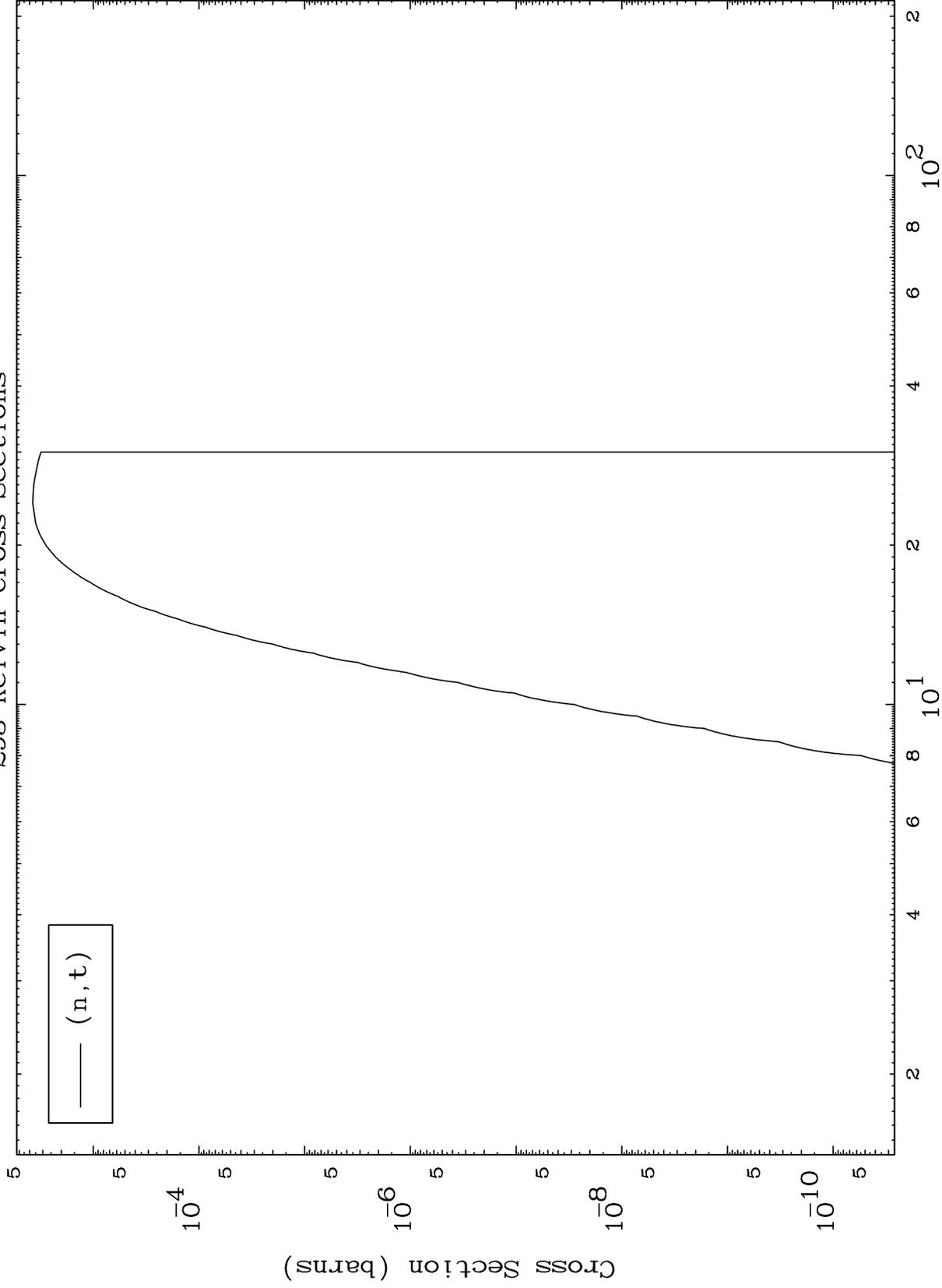
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

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11

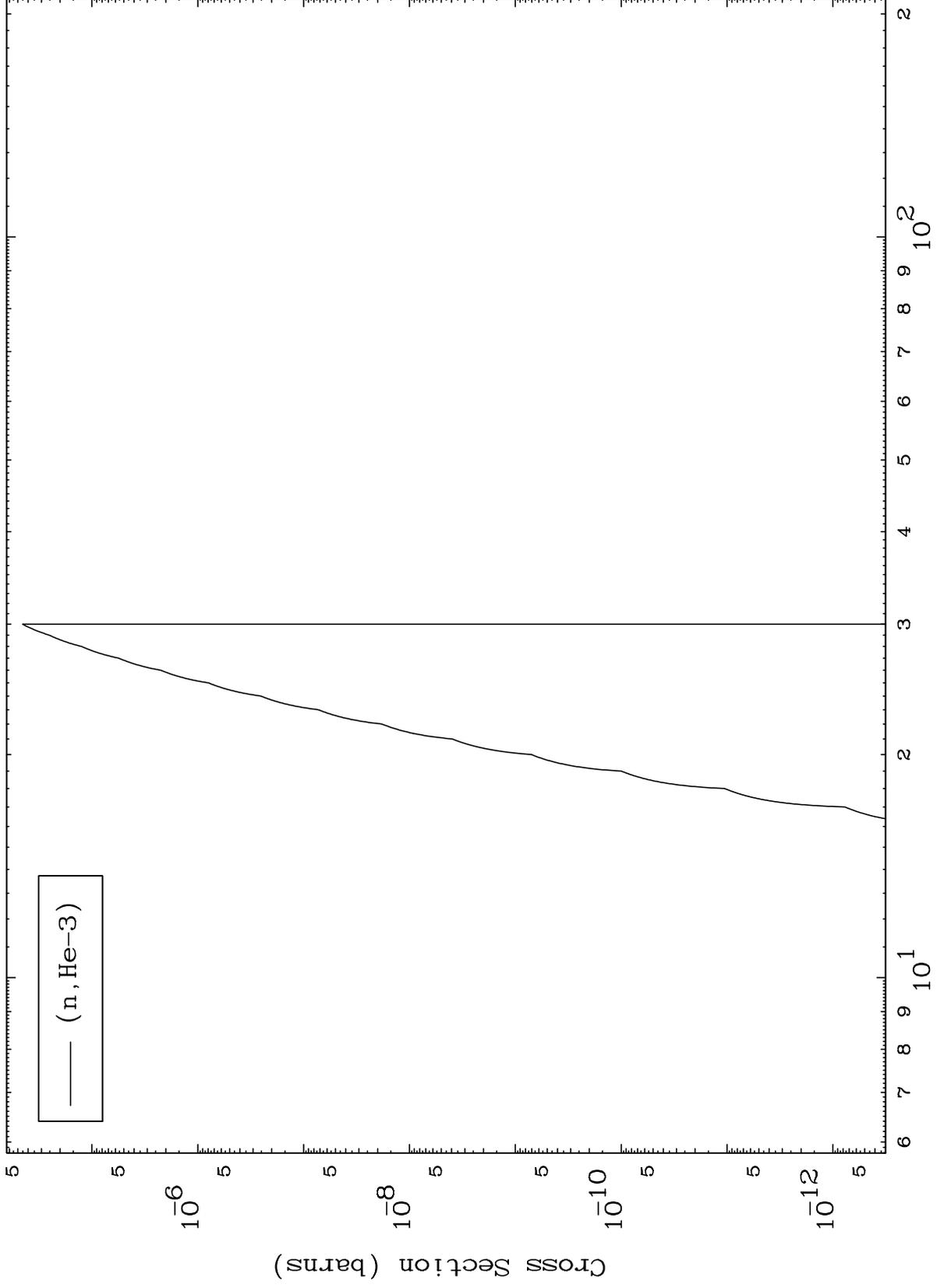
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

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12

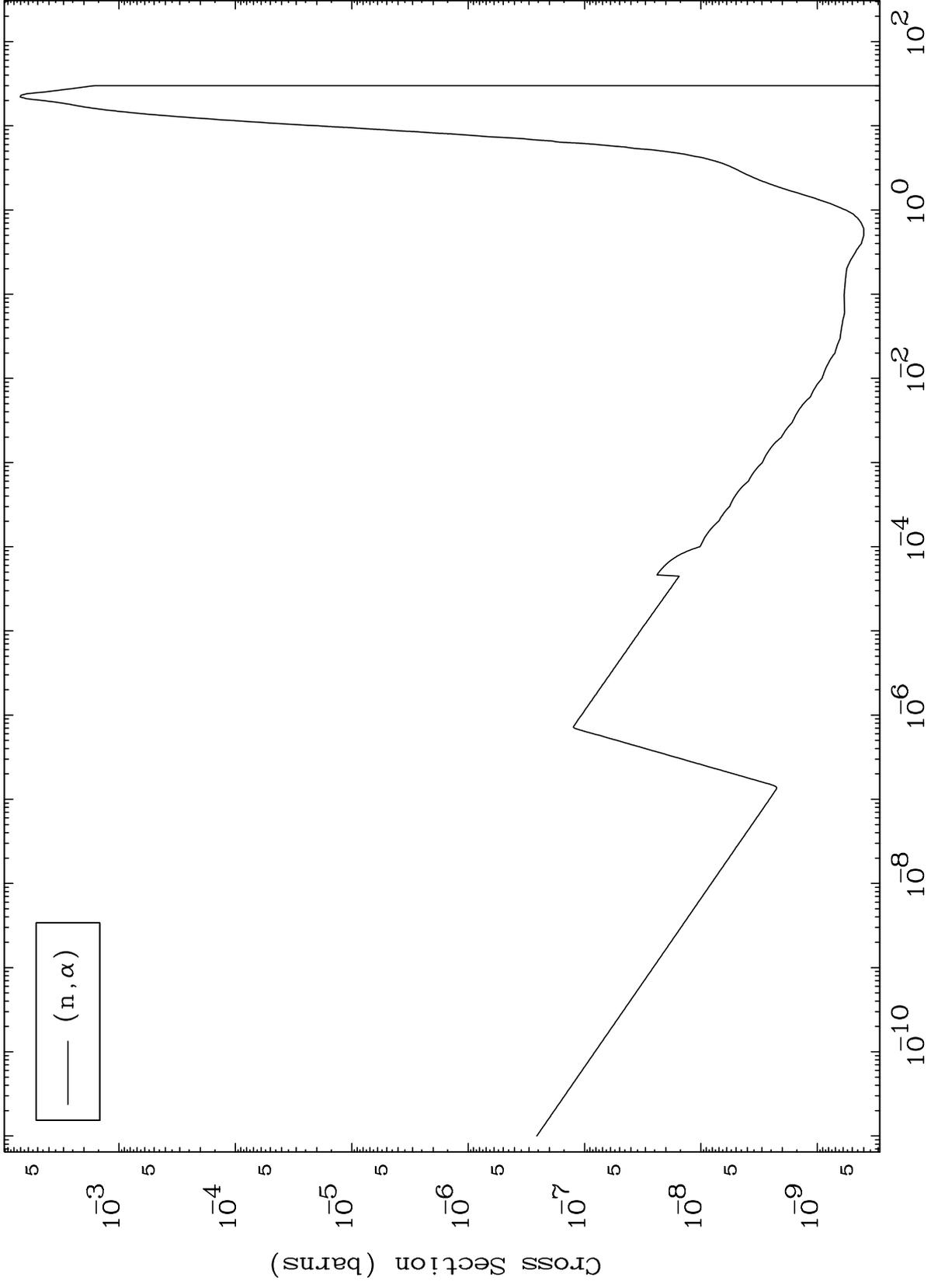
Incident Energy (MeV)

87-Fr-224

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

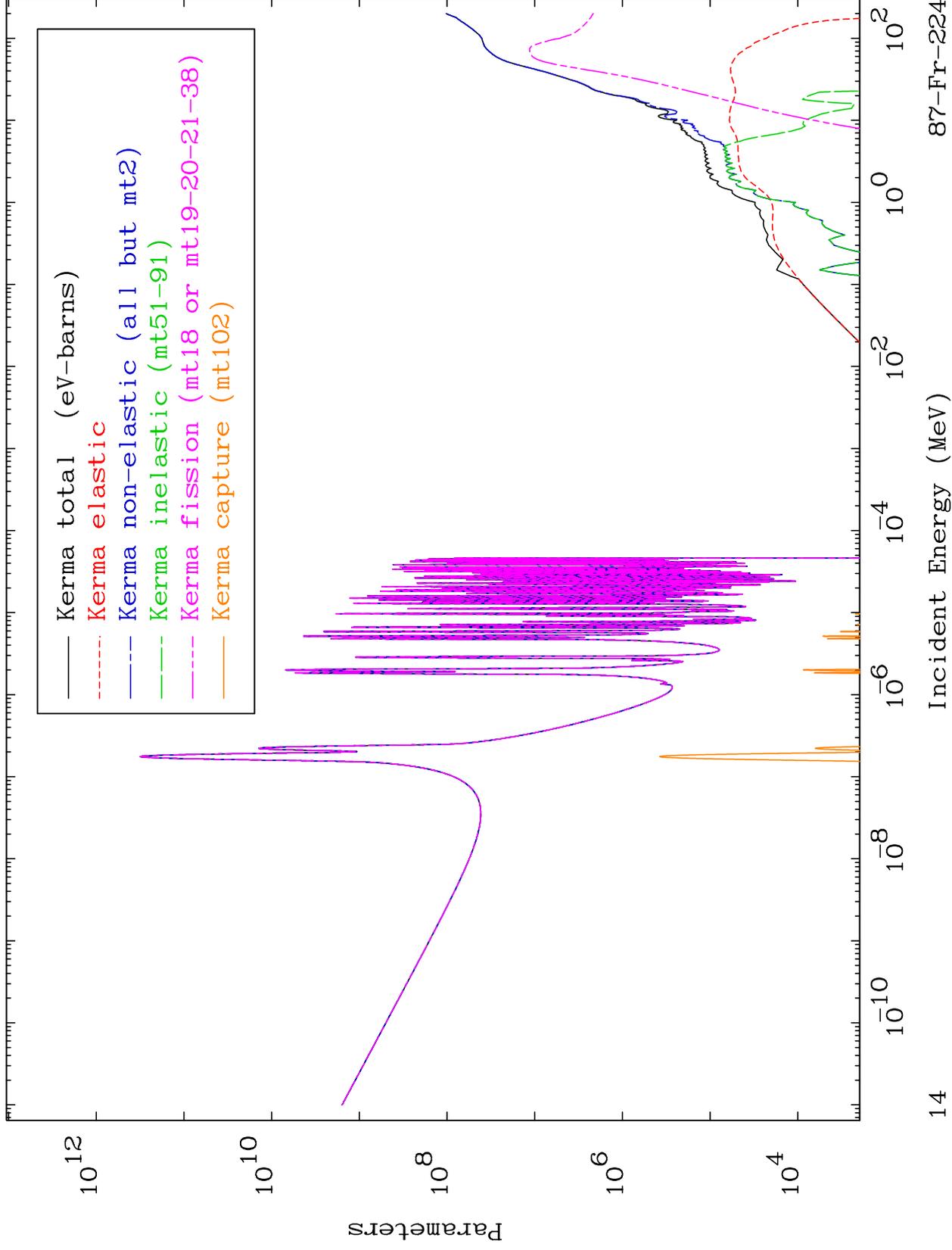
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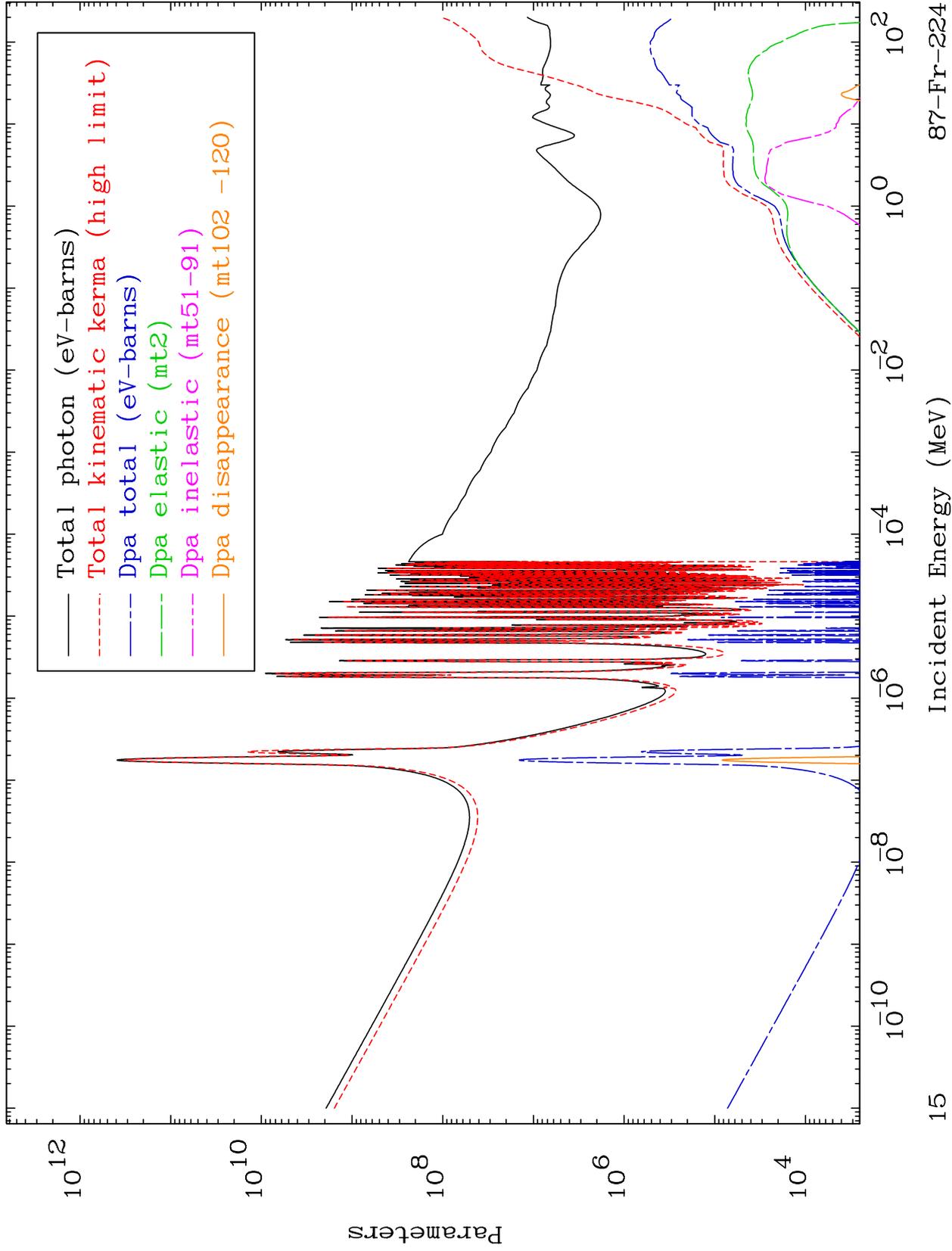


13

Incident Energy (MeV)

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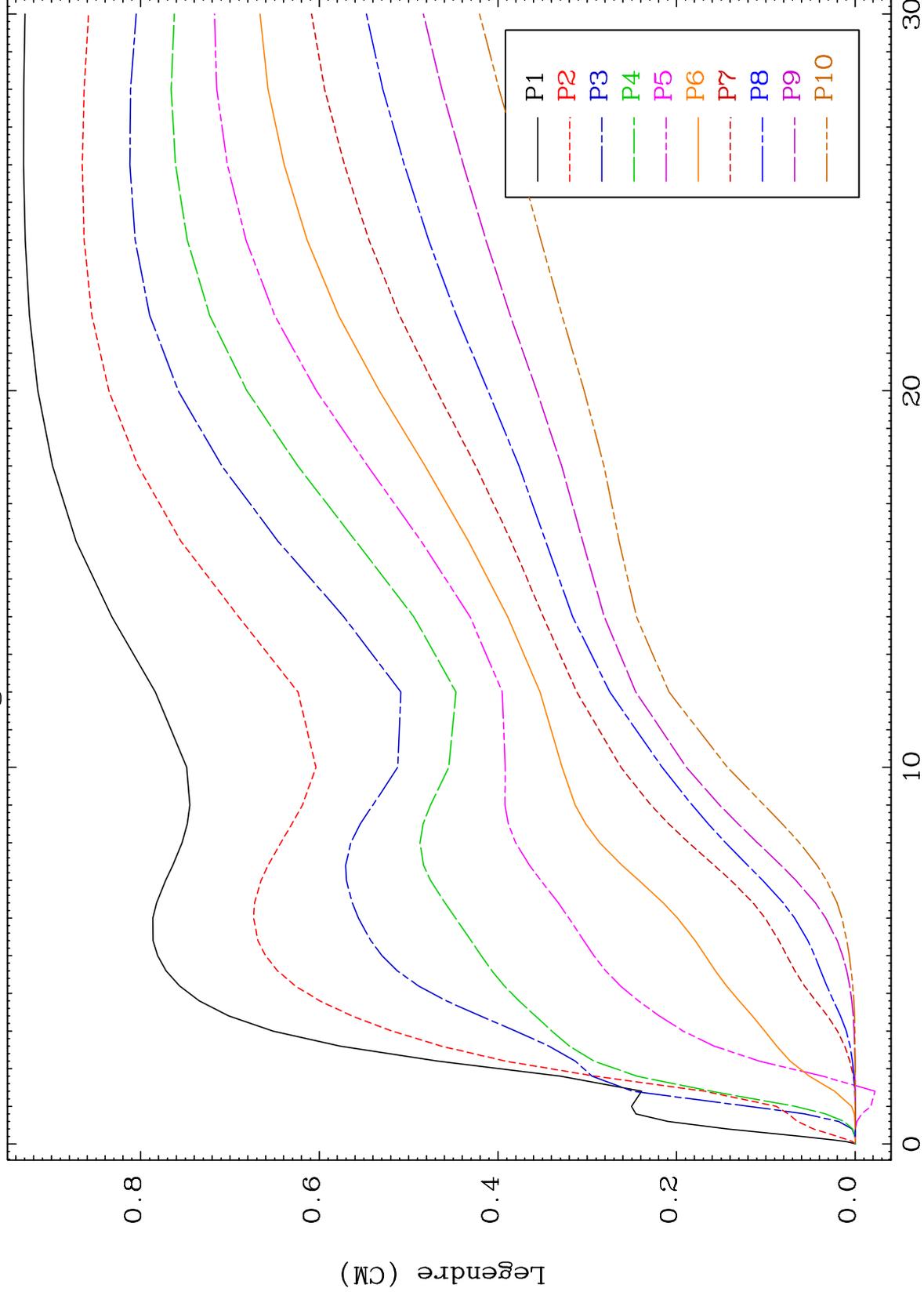




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Elastic Legendre Coefficients

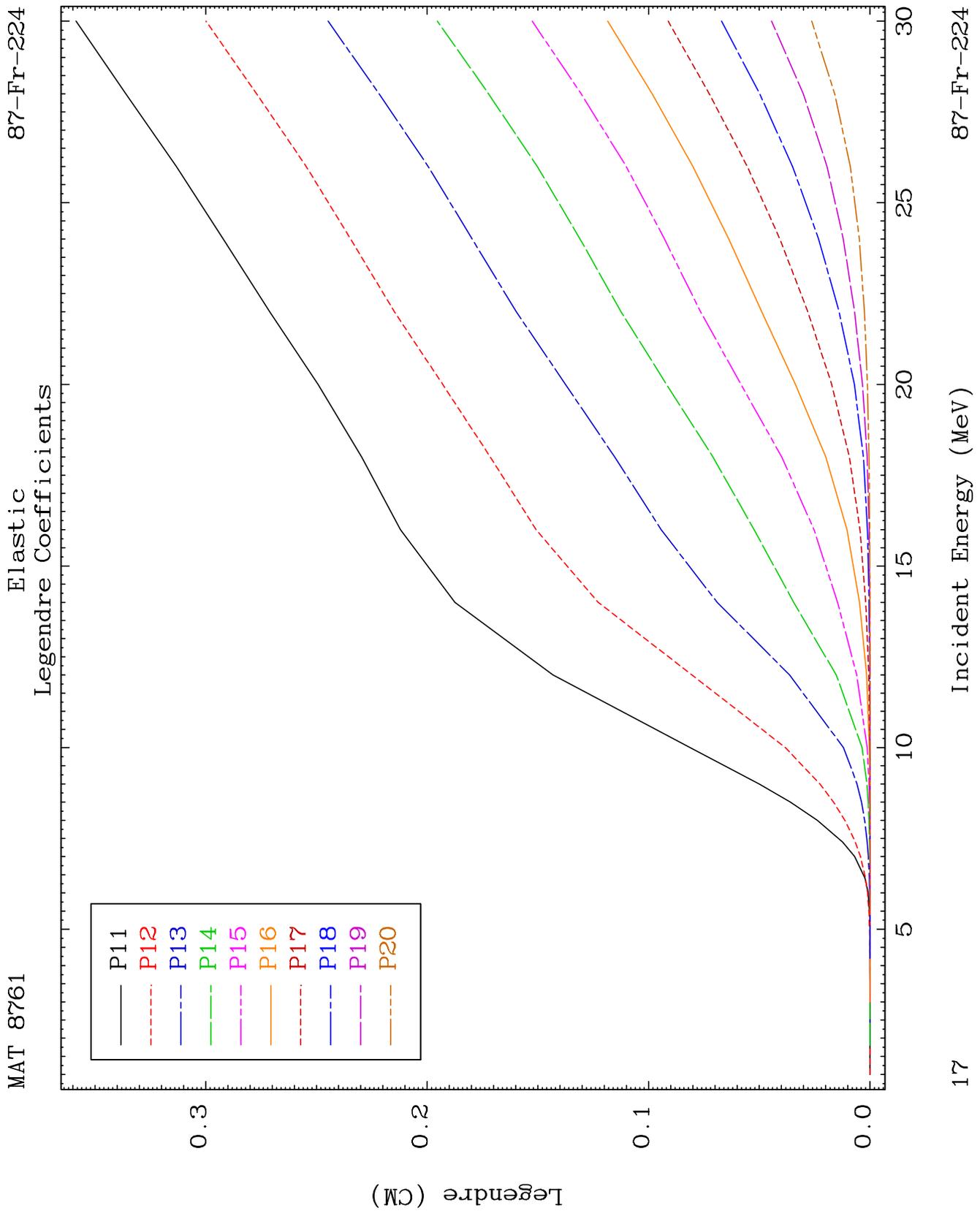
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16

Incident Energy (MeV)

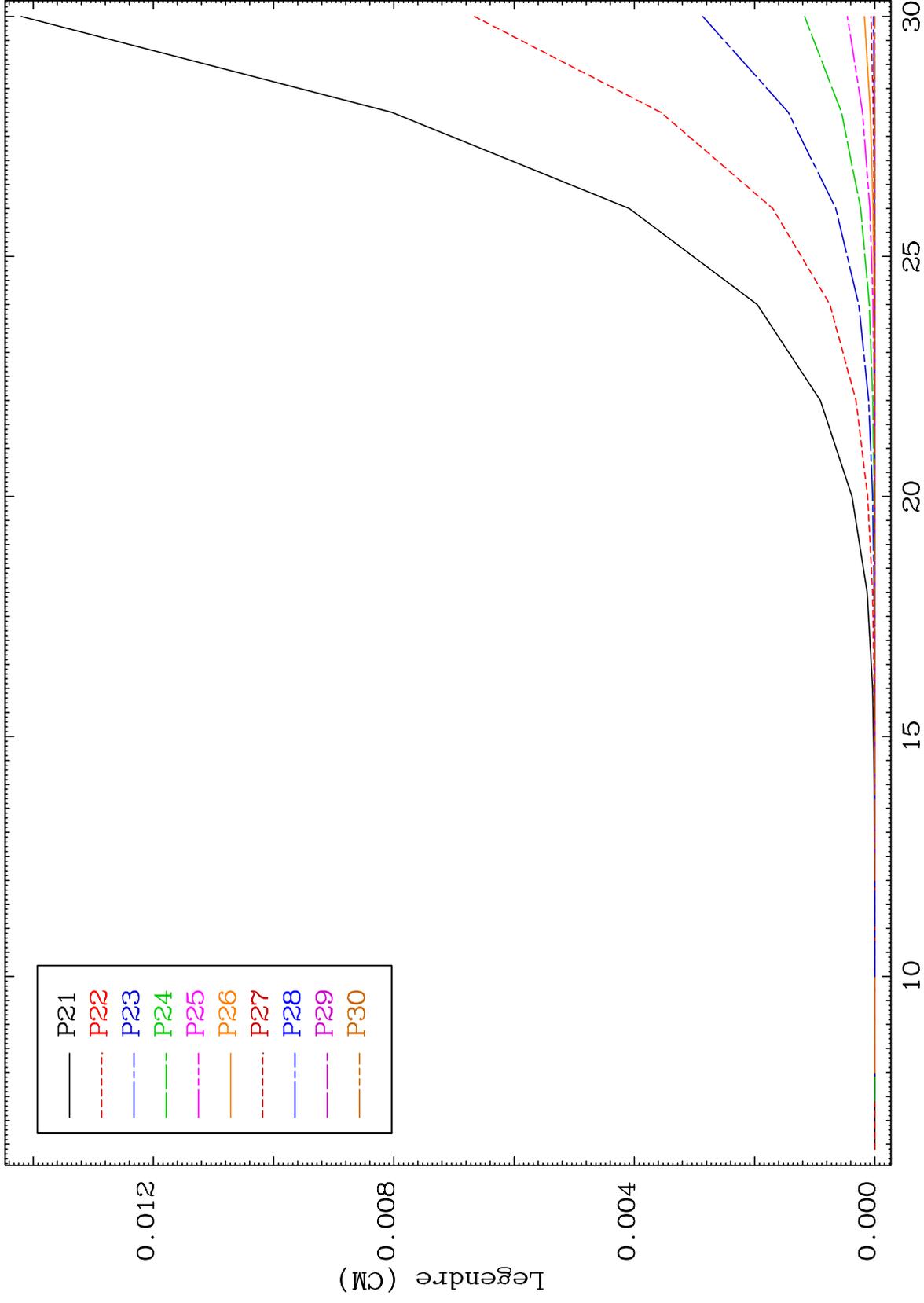
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Elastic  
Legendre Coefficients

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18

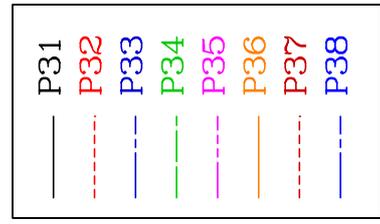
Incident Energy (MeV)

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Elastic  
Legendre Coefficients

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$\times 10^{-7}$

Legendre (CM)

6

4

2

0

16

18

20

22

24

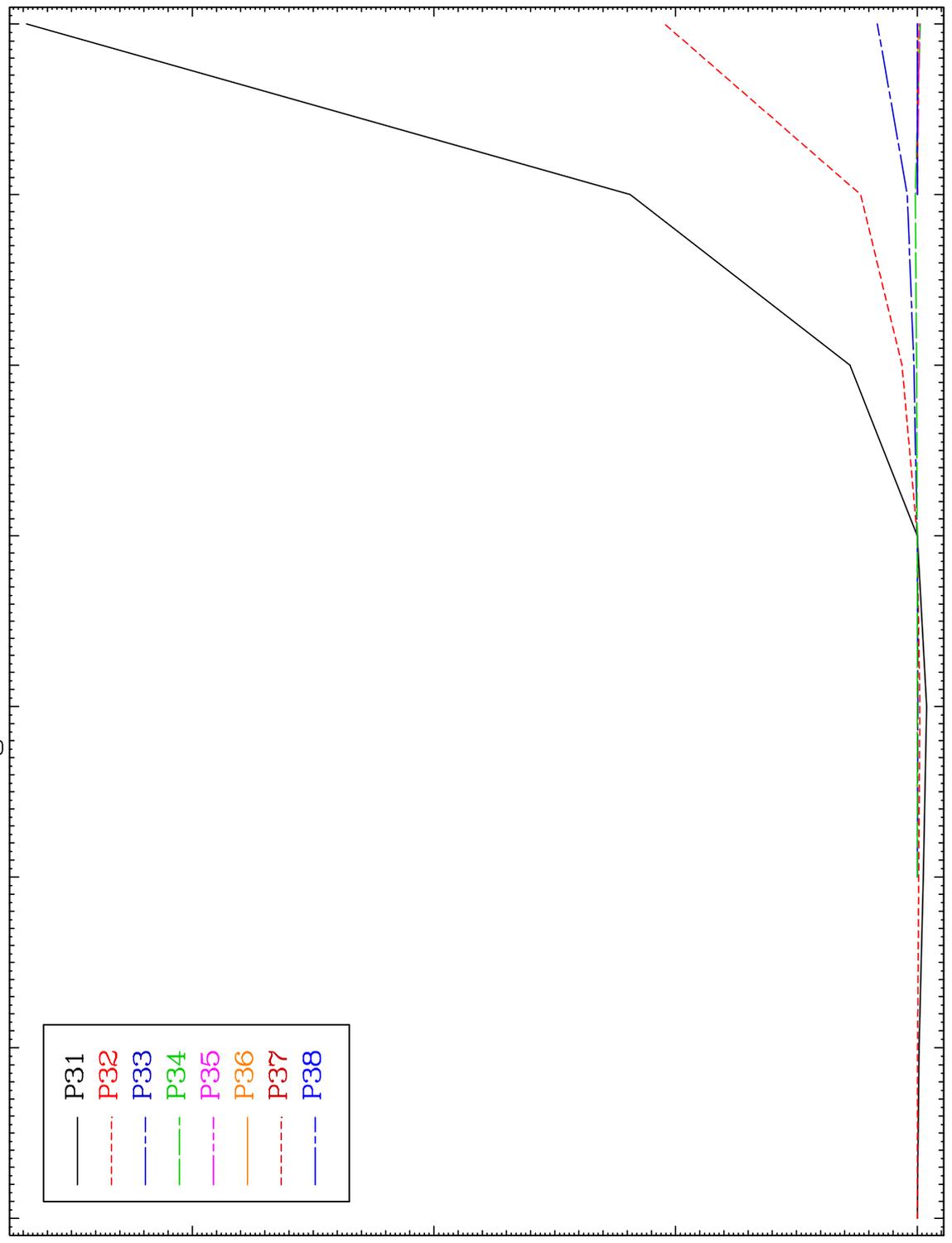
26

28

30

Incident Energy (MeV)

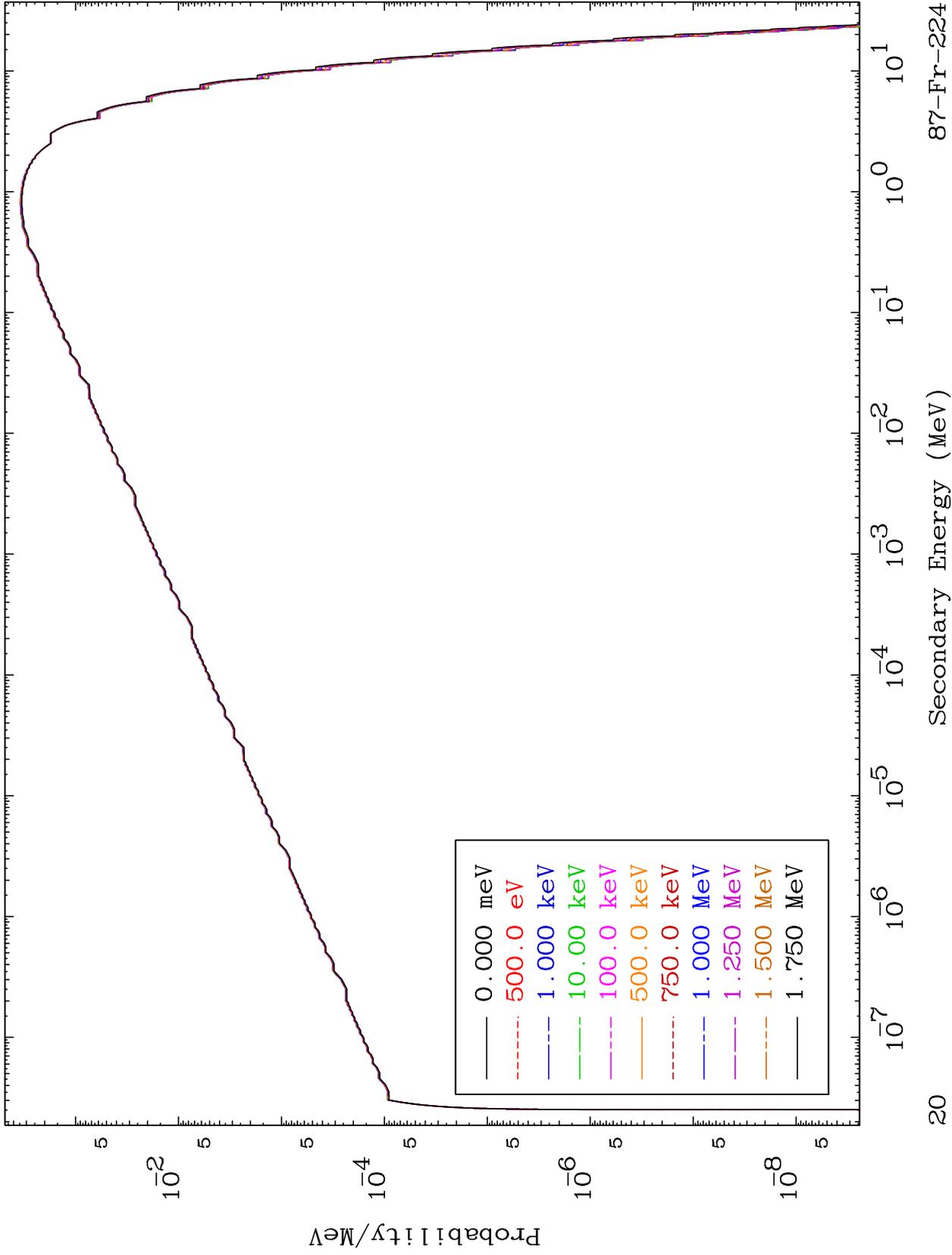
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### Fission Energy Distribution

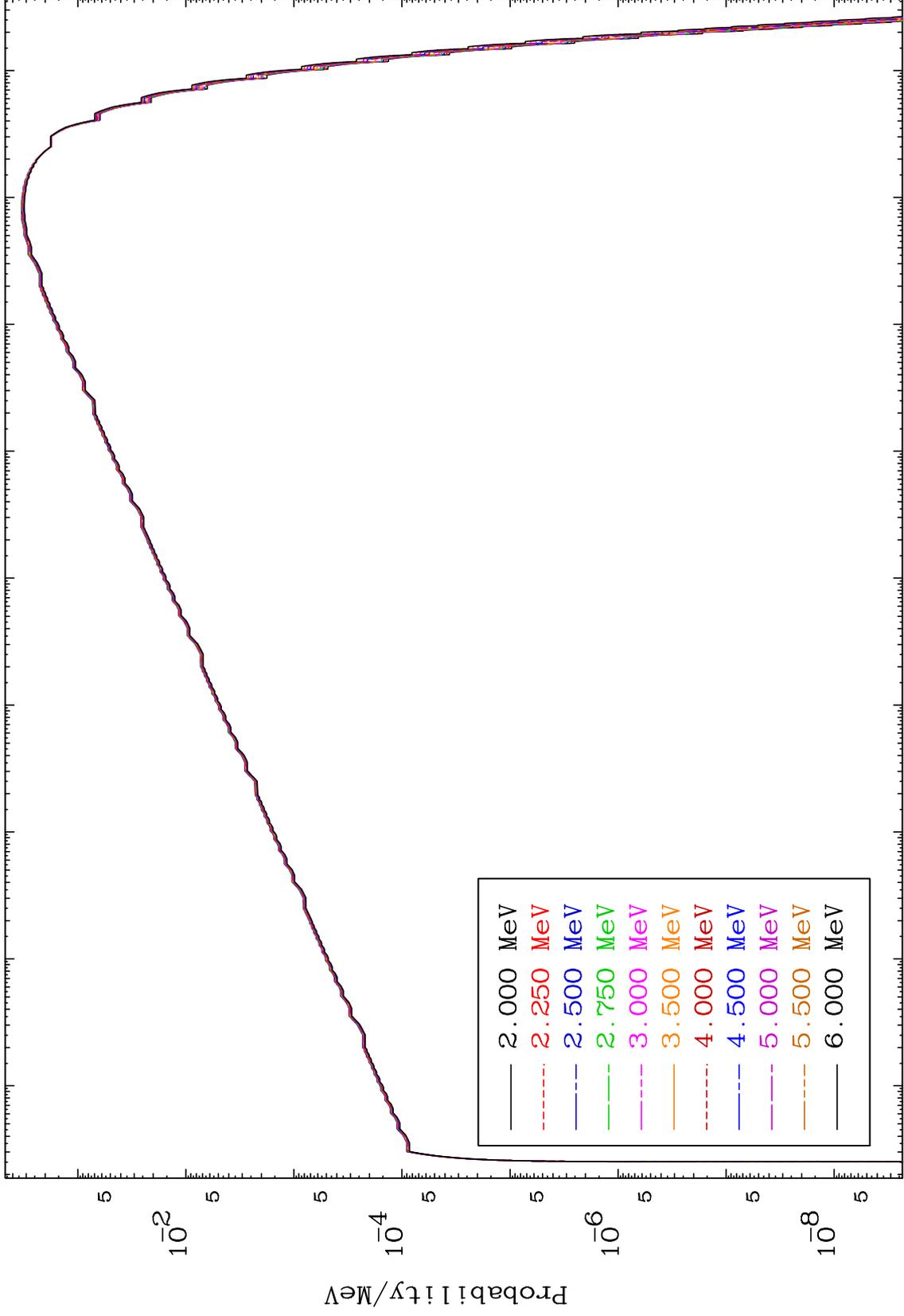
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# Fission Energy Distribution

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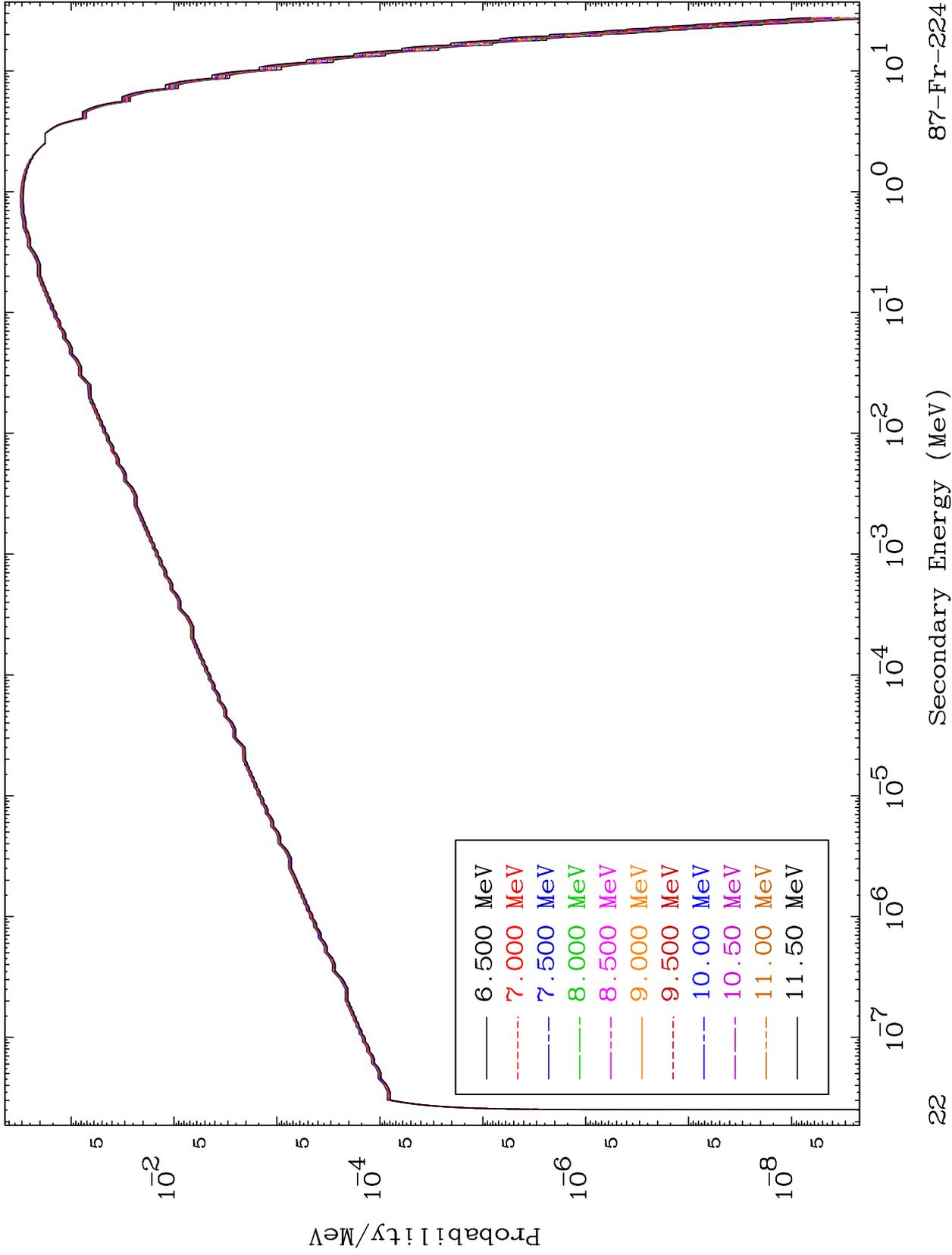


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### Fission Energy Distribution

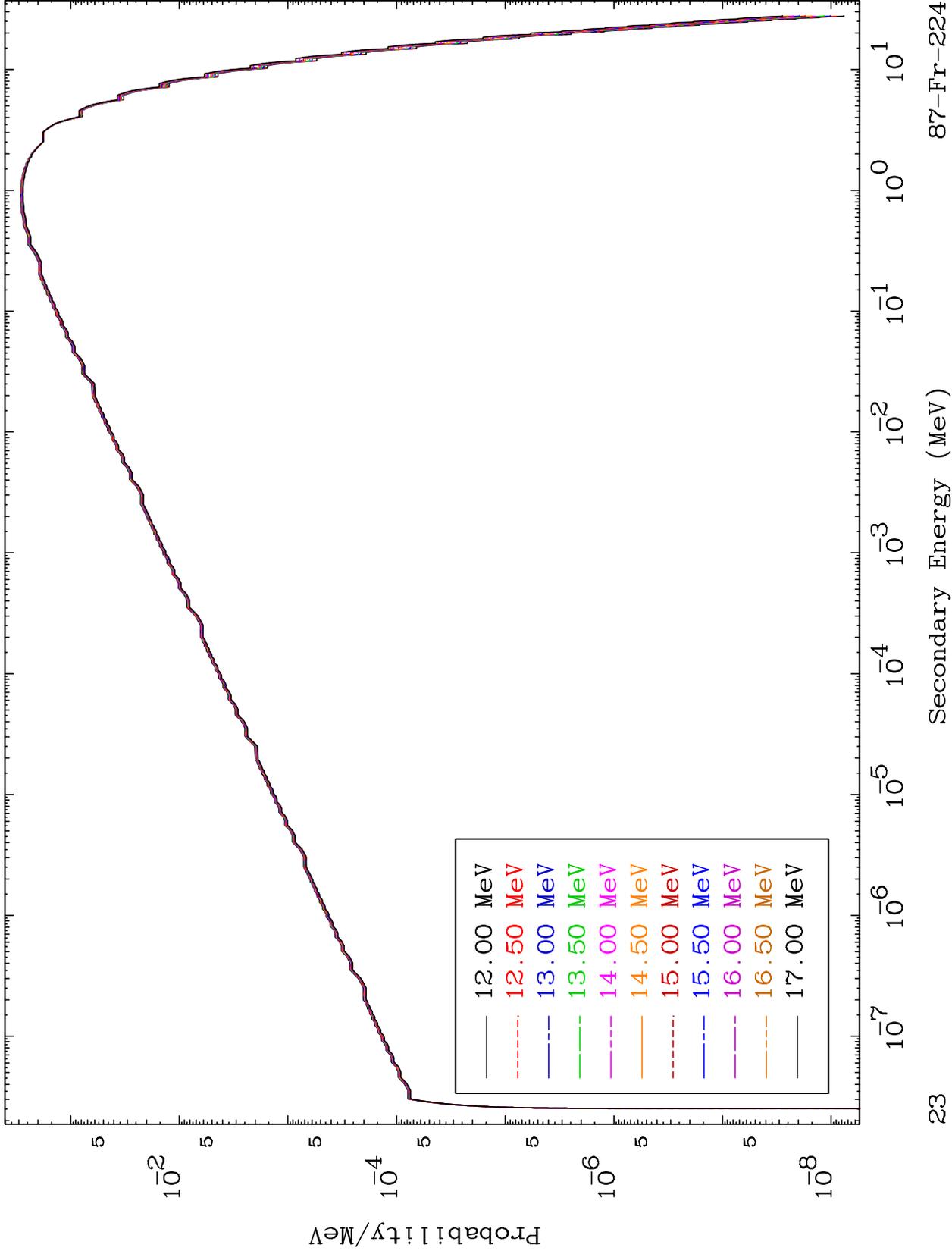
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### Fission Energy Distribution

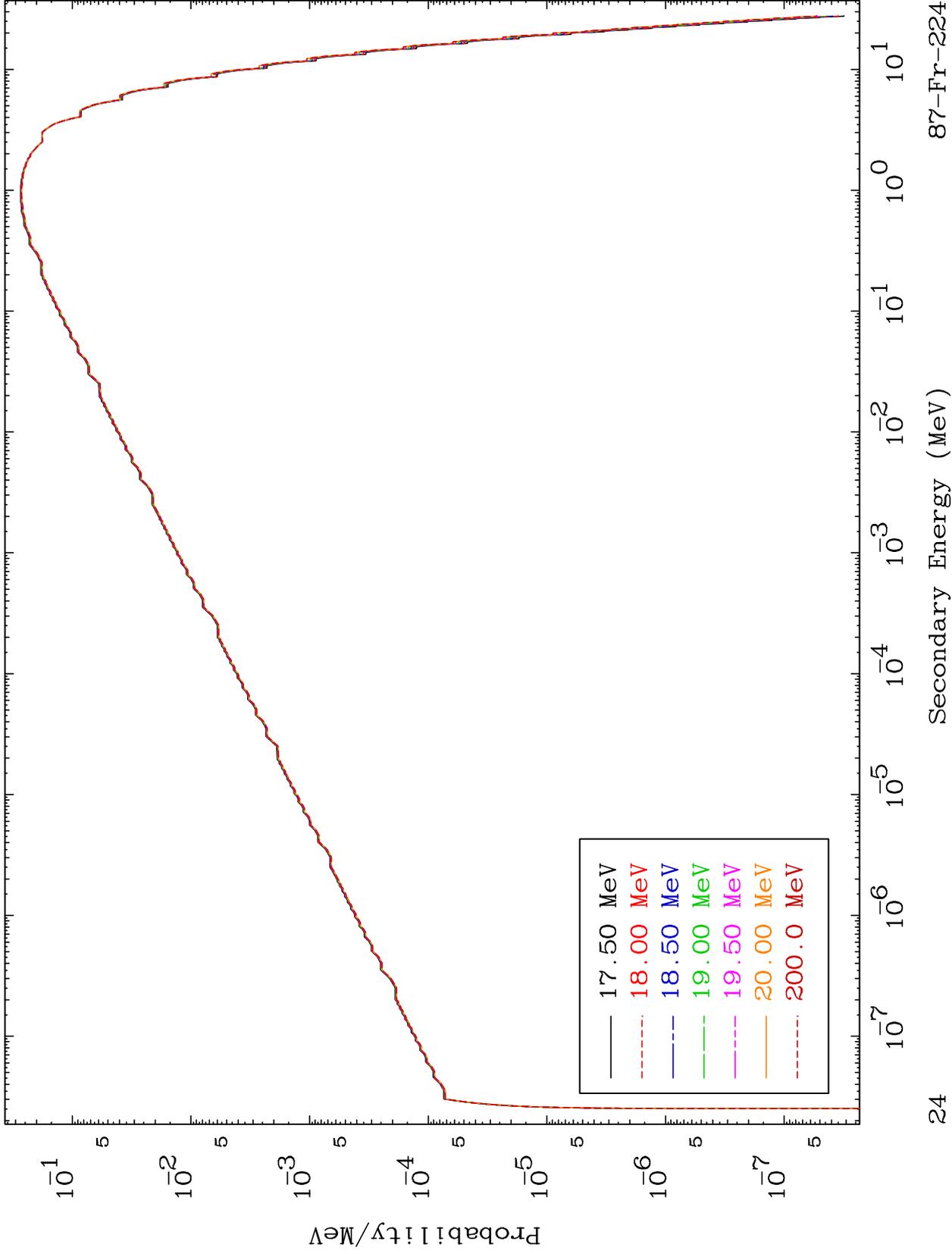
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### Fission Energy Distribution

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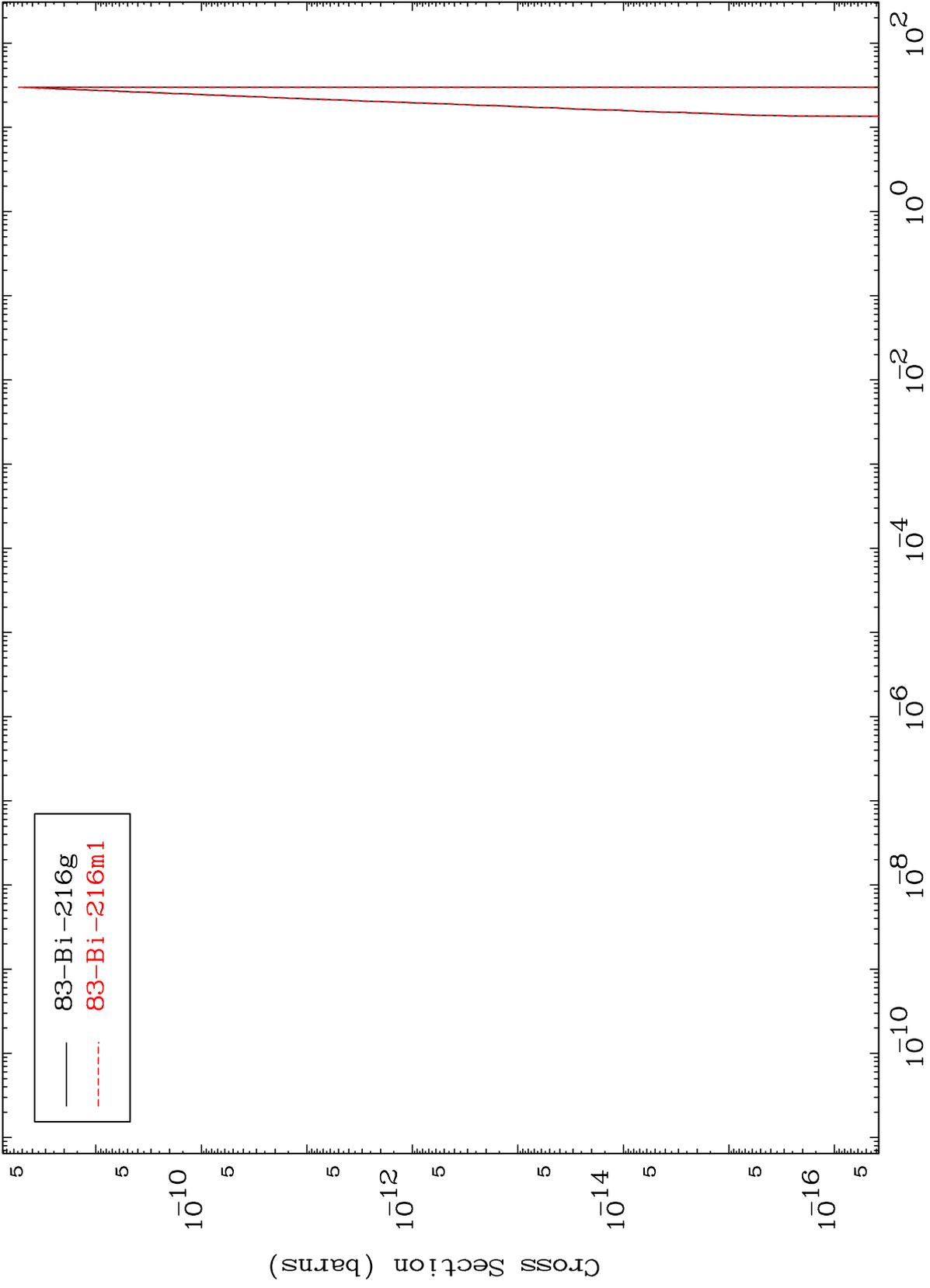


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(n,n') 2 $\alpha$

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

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MAT 8761

(n,2n) 2 $\alpha$

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Radionuclide Production Cross Section

