

Program EVALPLOT  
(Version 2017-1)

by

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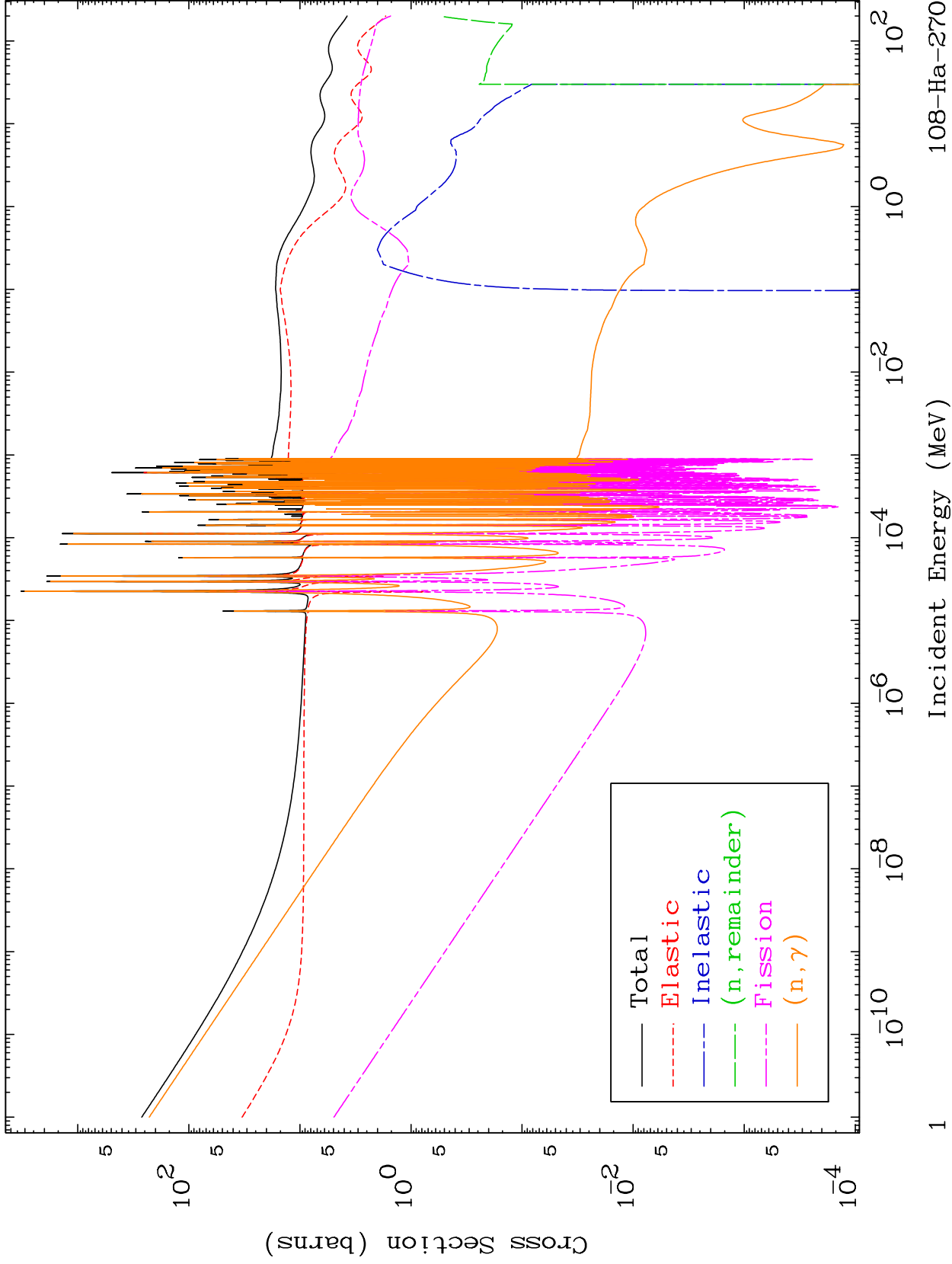
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 870

Major  
293 Kelvin Cross Sections

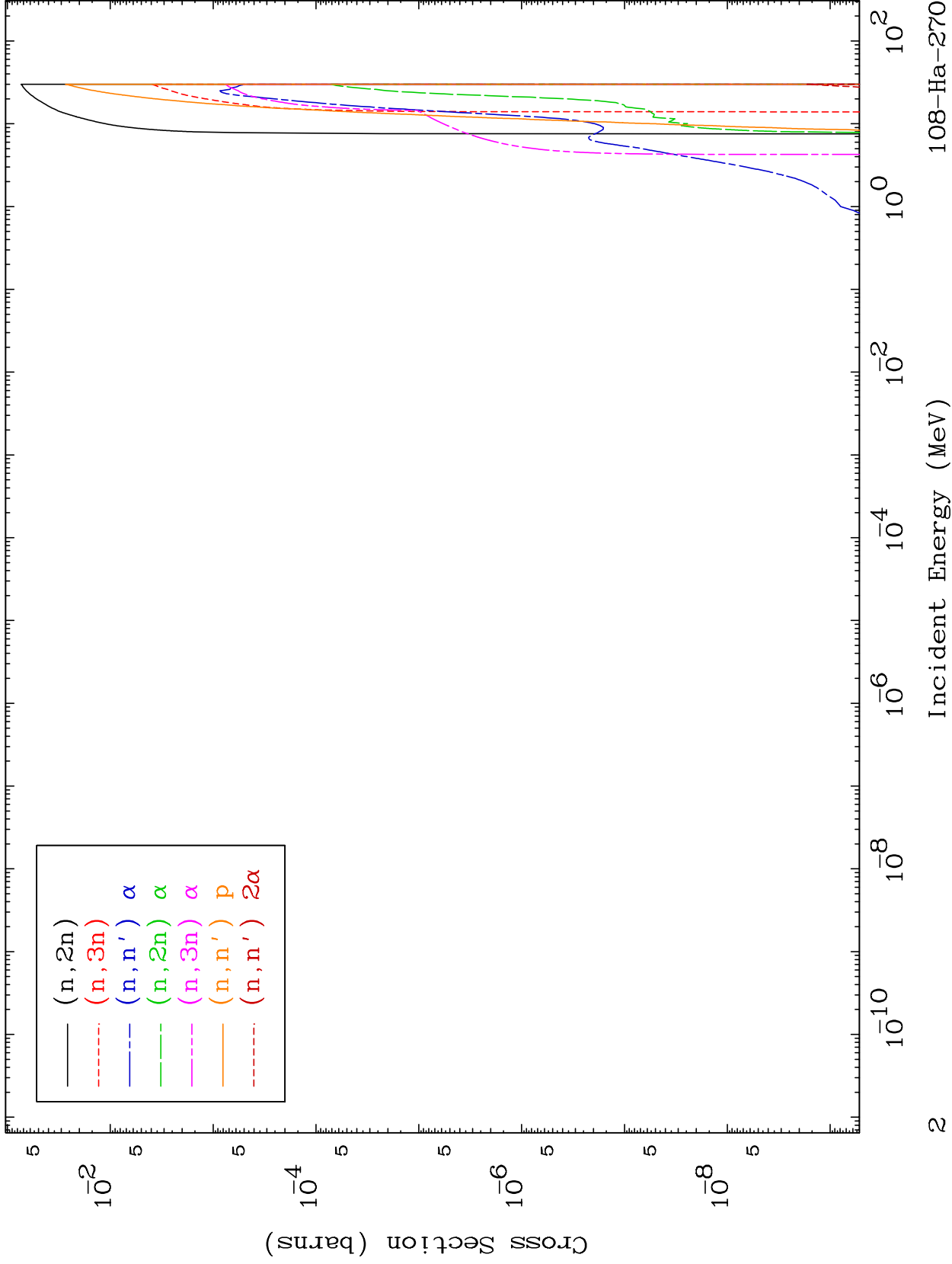
108-Ha-270



MAT 870

Neutron Production  
293 Kelvin Cross Sections

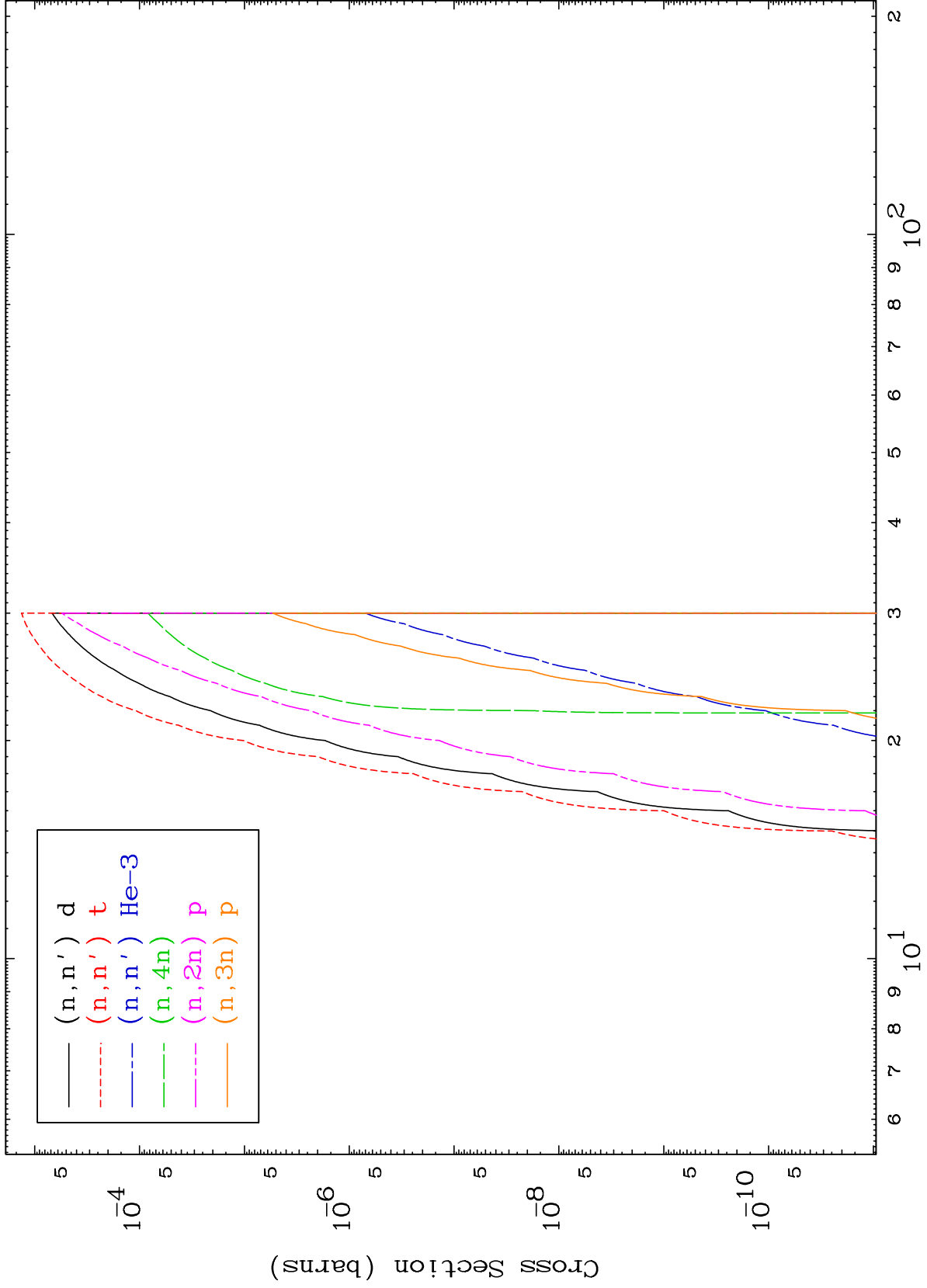
108-Ha-270

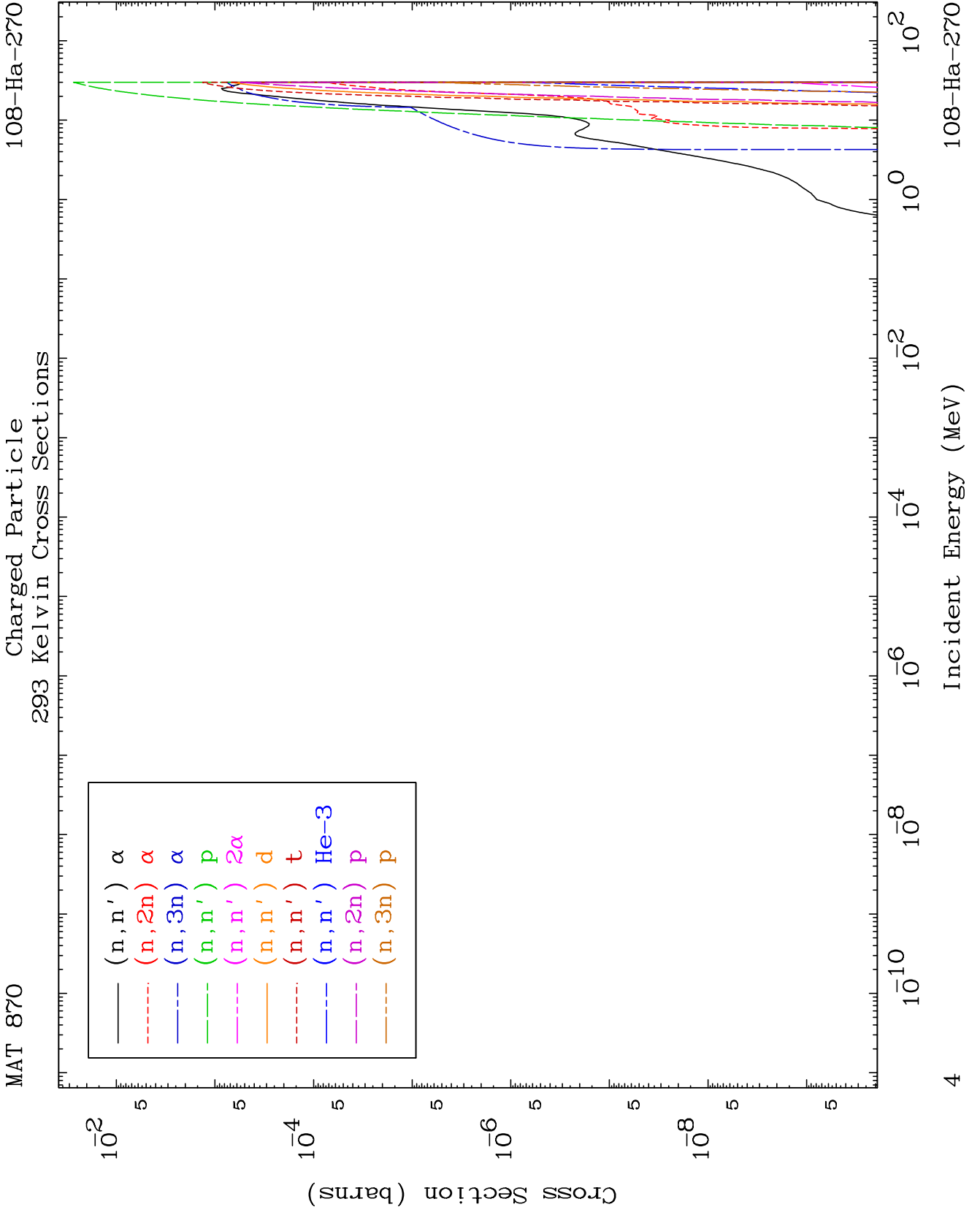


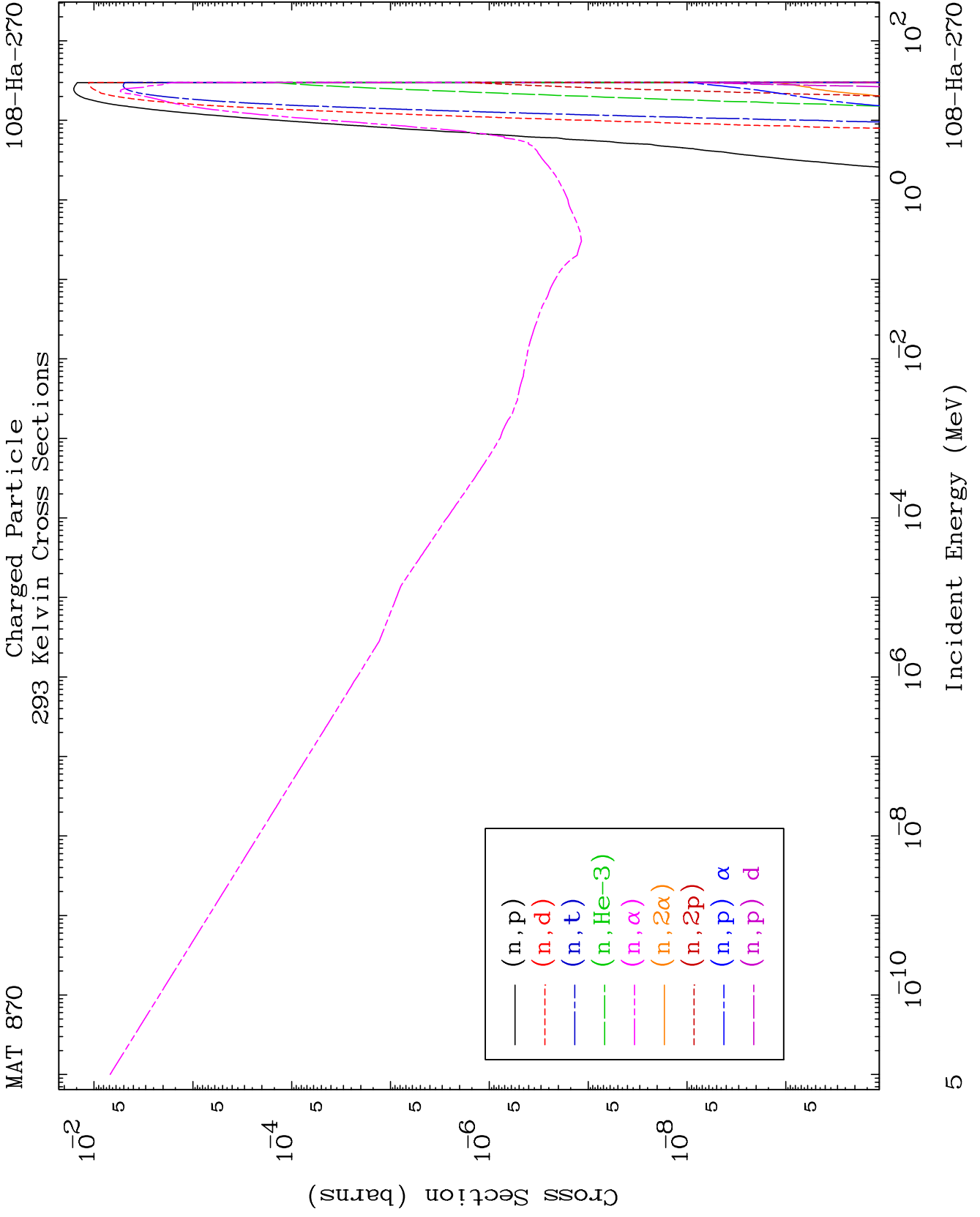
MAT 870

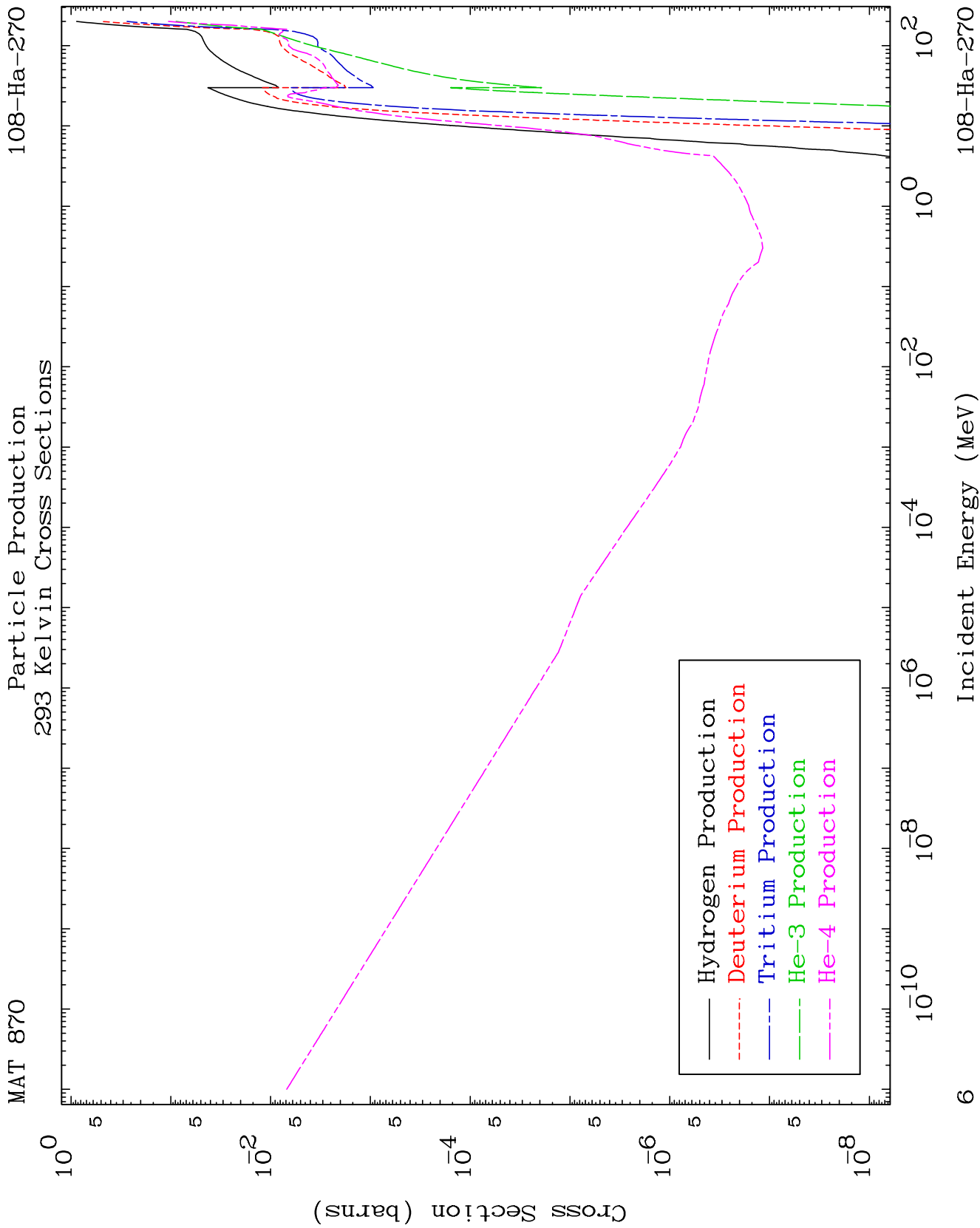
Neutron Production  
293 Kelvin Cross Sections

108-Ha-270





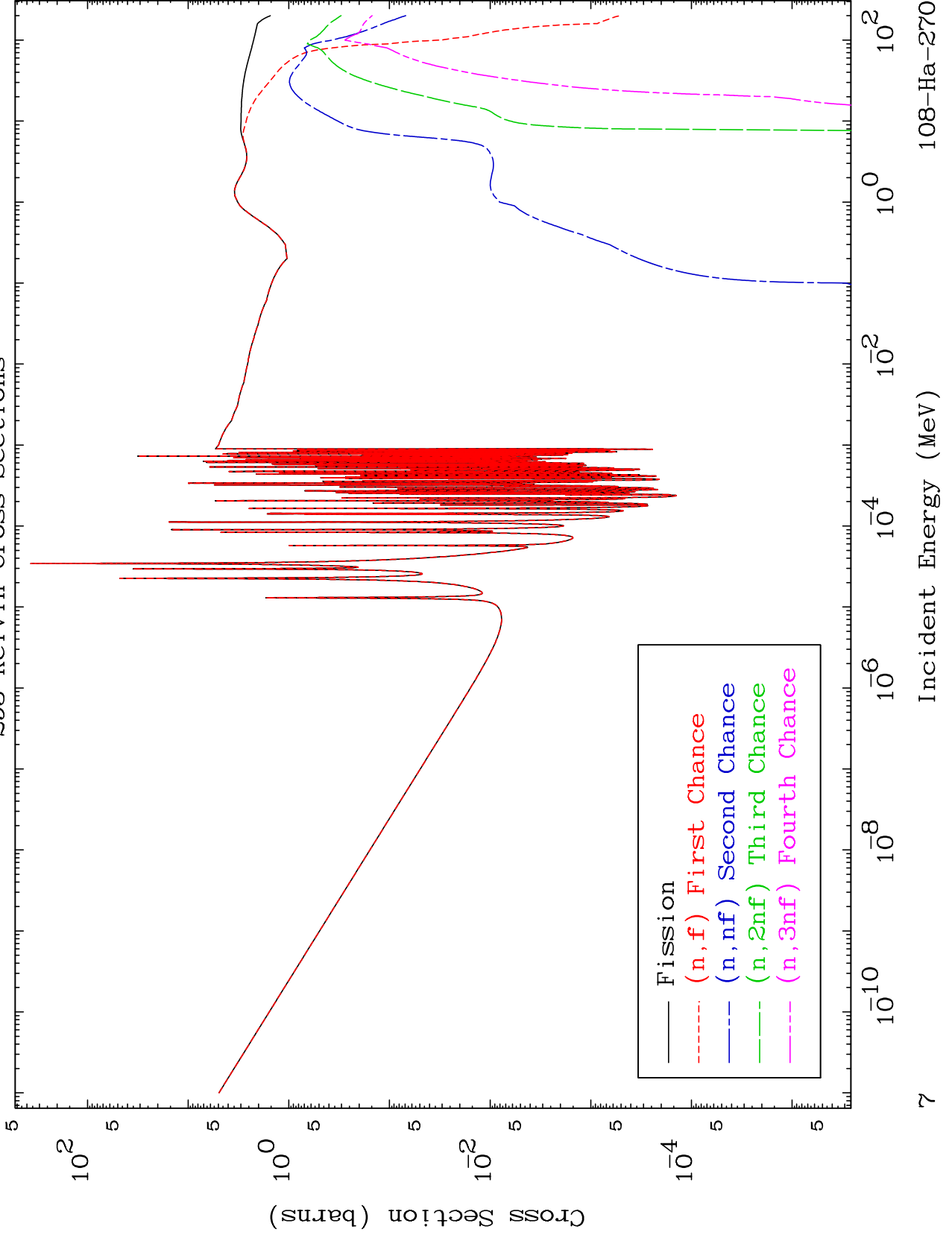




MAT 870

Fission  
293 Kelvin Cross Sections

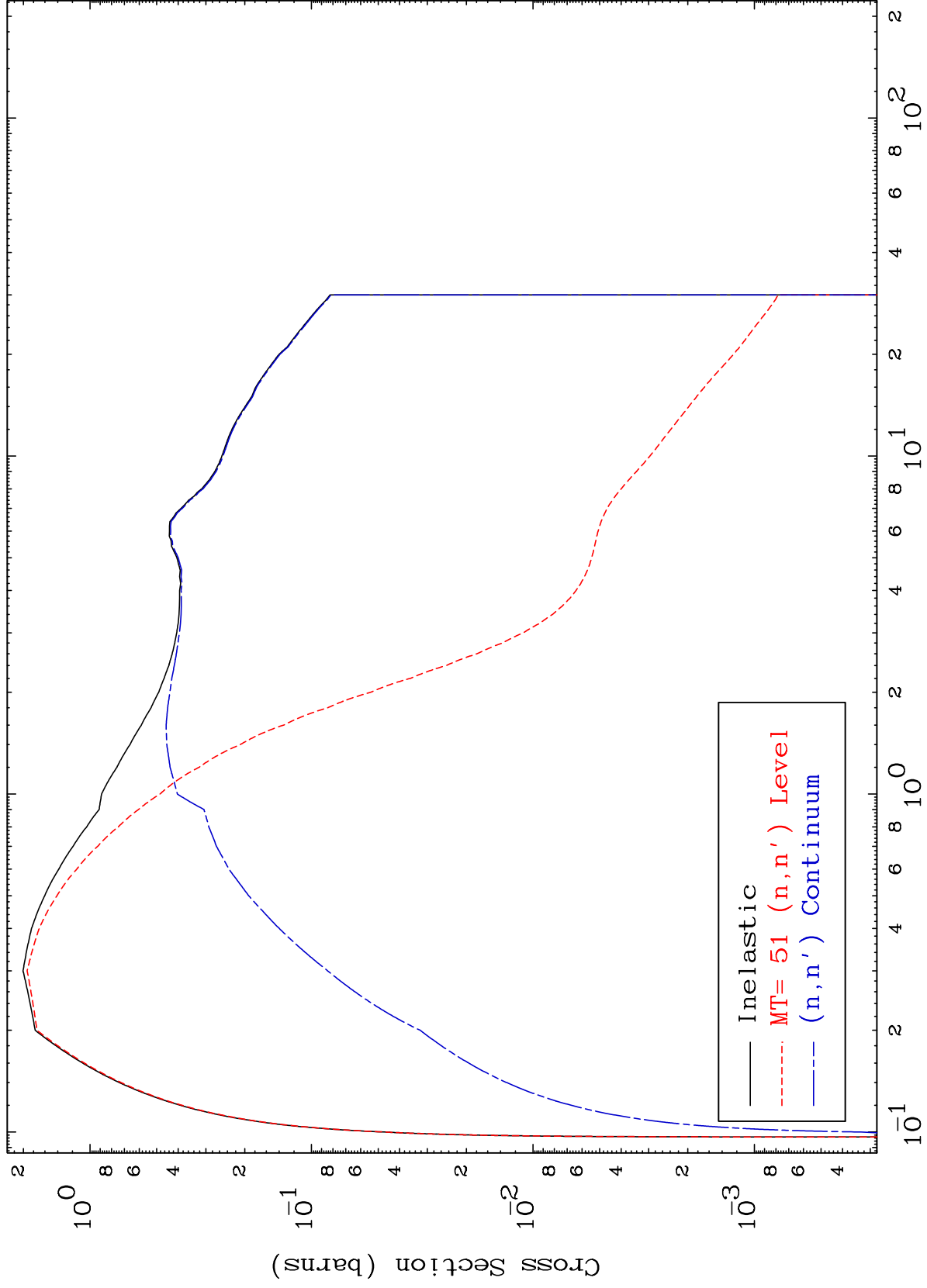
108-Ha-270



MAT 870

(n,n') Level  
293 Kelvin Cross Sections

108-Ha-270



Incident Energy (MeV)

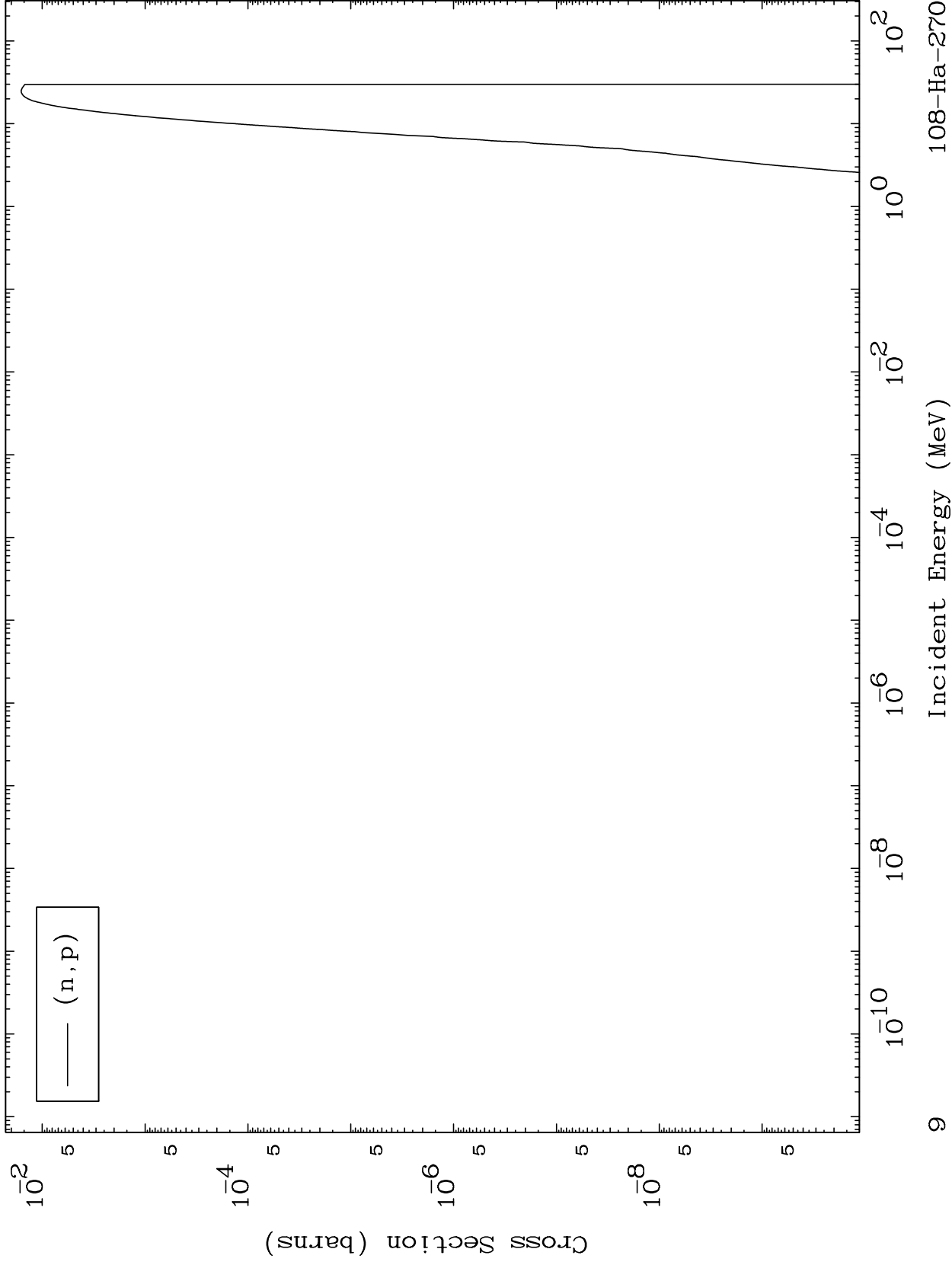
108-Ha-270

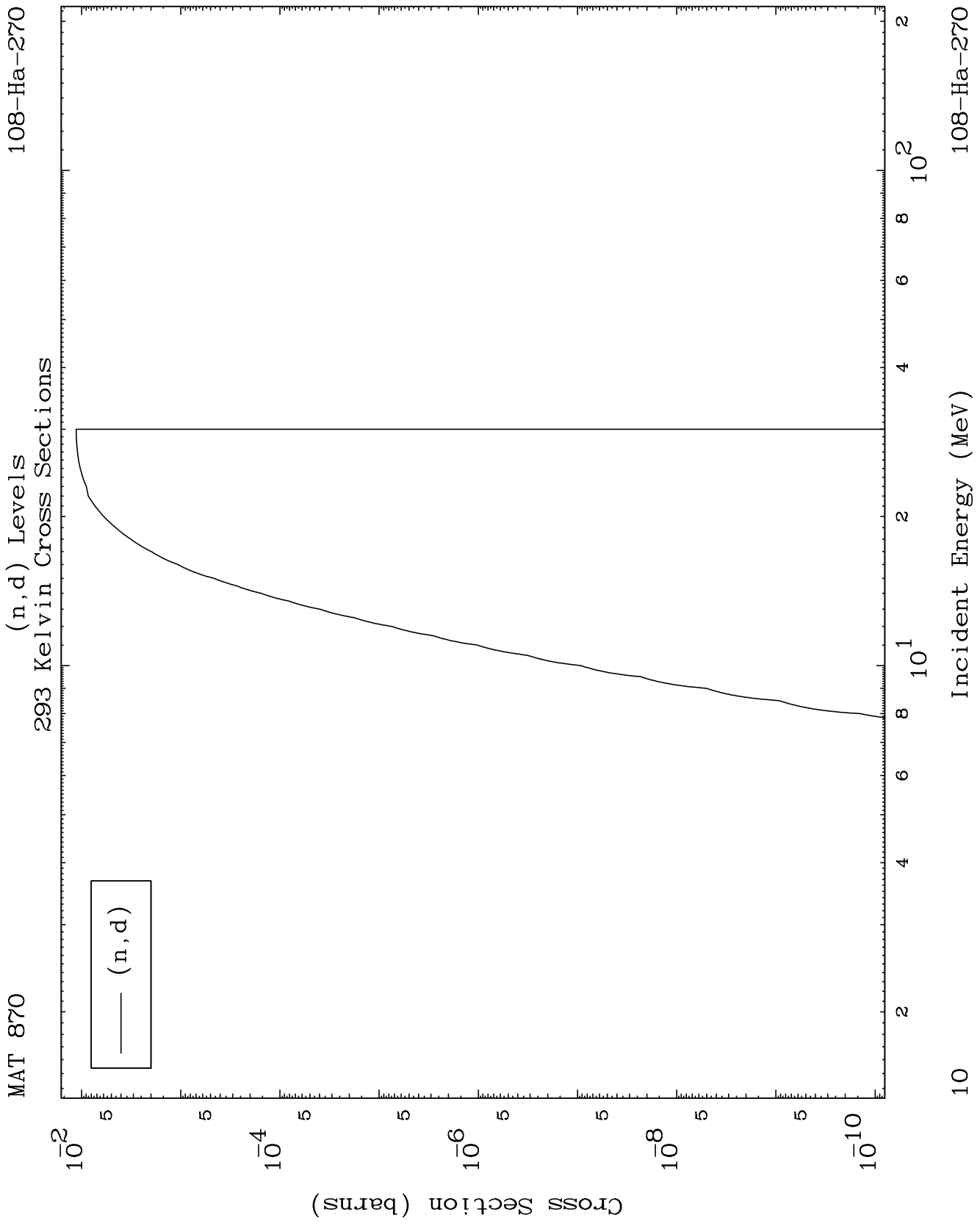
8

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(n,p) Levels  
293 Kelvin Cross Sections

108-Ha-270

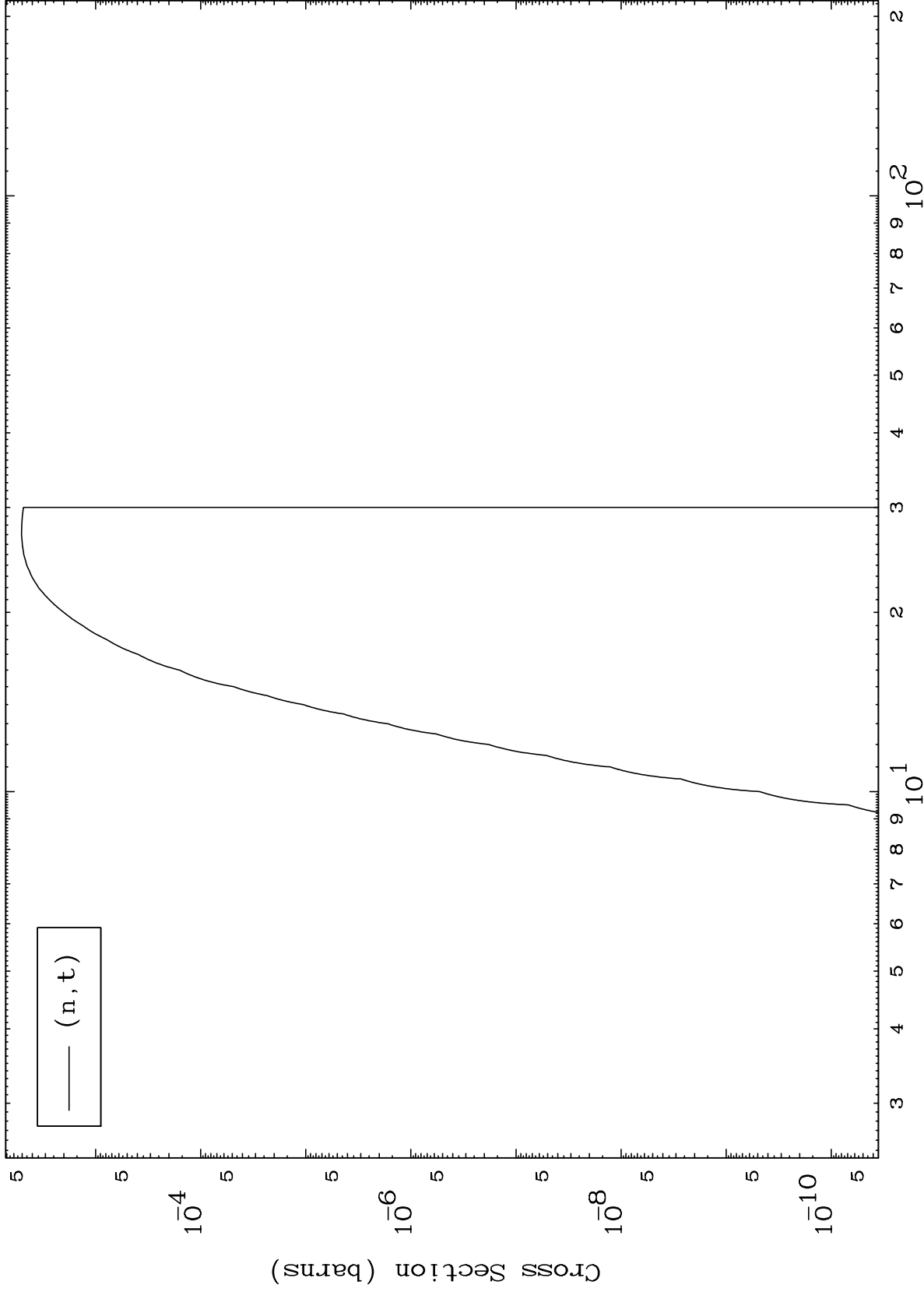




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(n,t) Levels  
293 Kelvin Cross Sections

108-Ha-270



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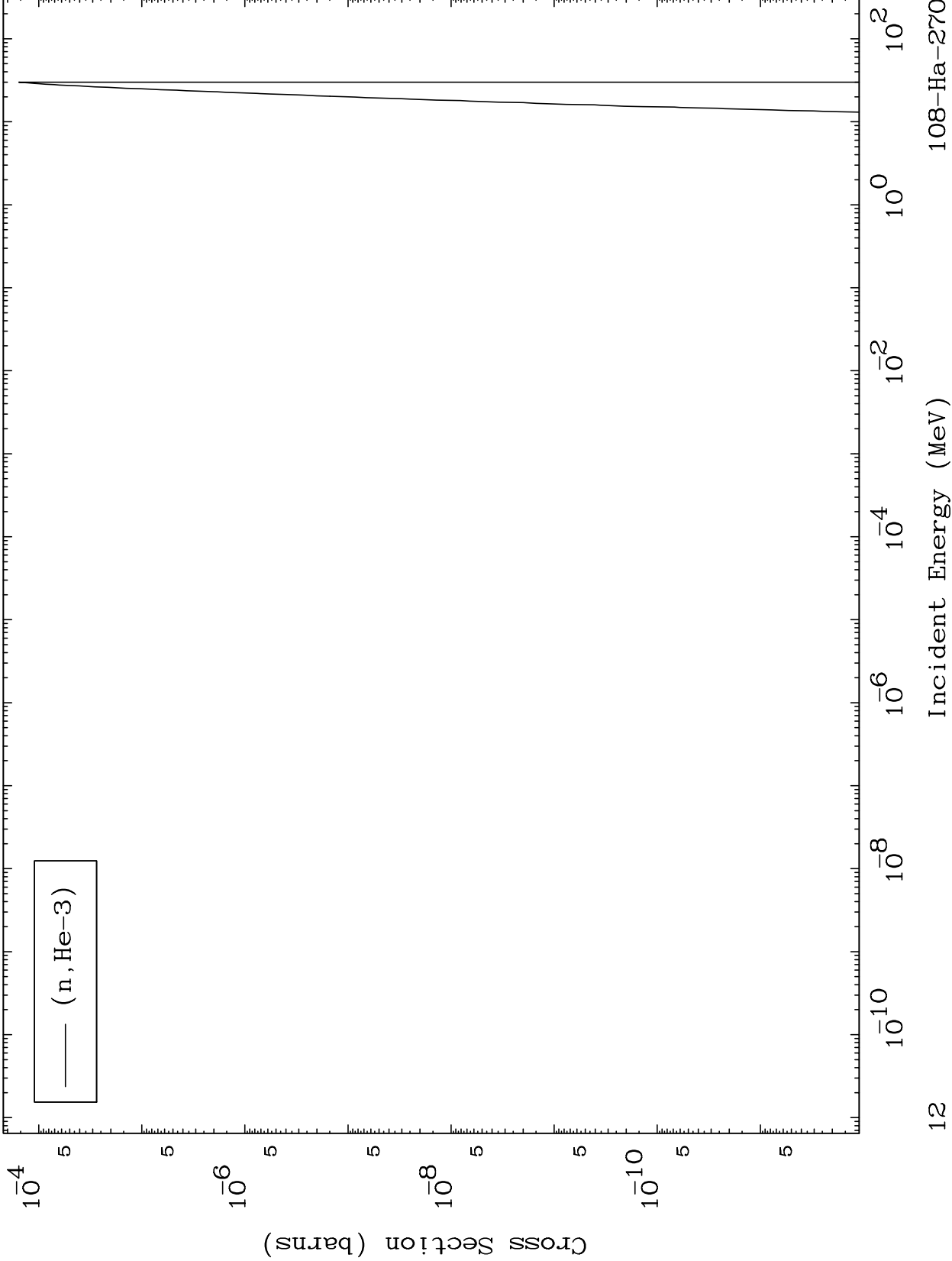
Incident Energy (MeV)

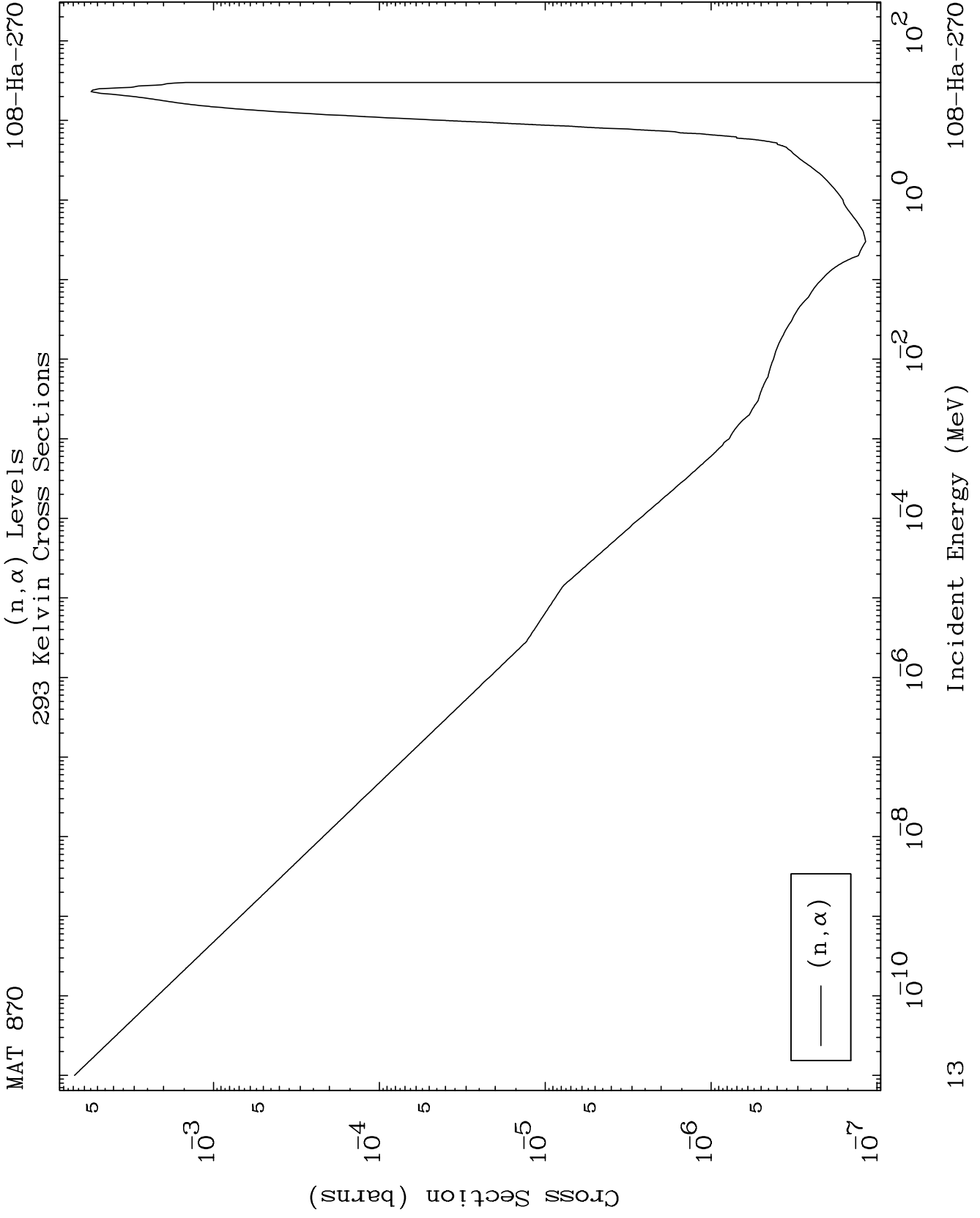
108-Ha-270

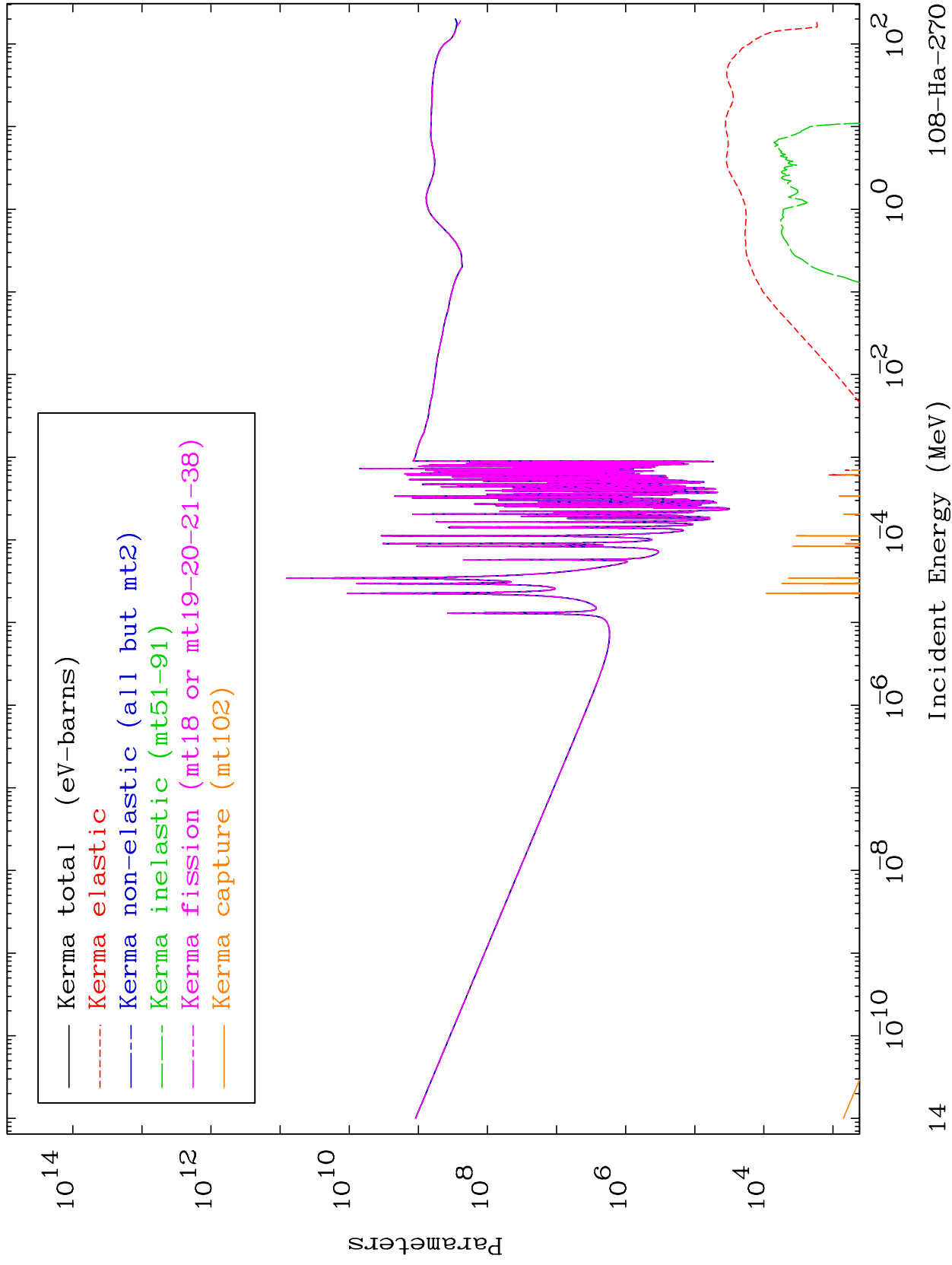
MAT 870

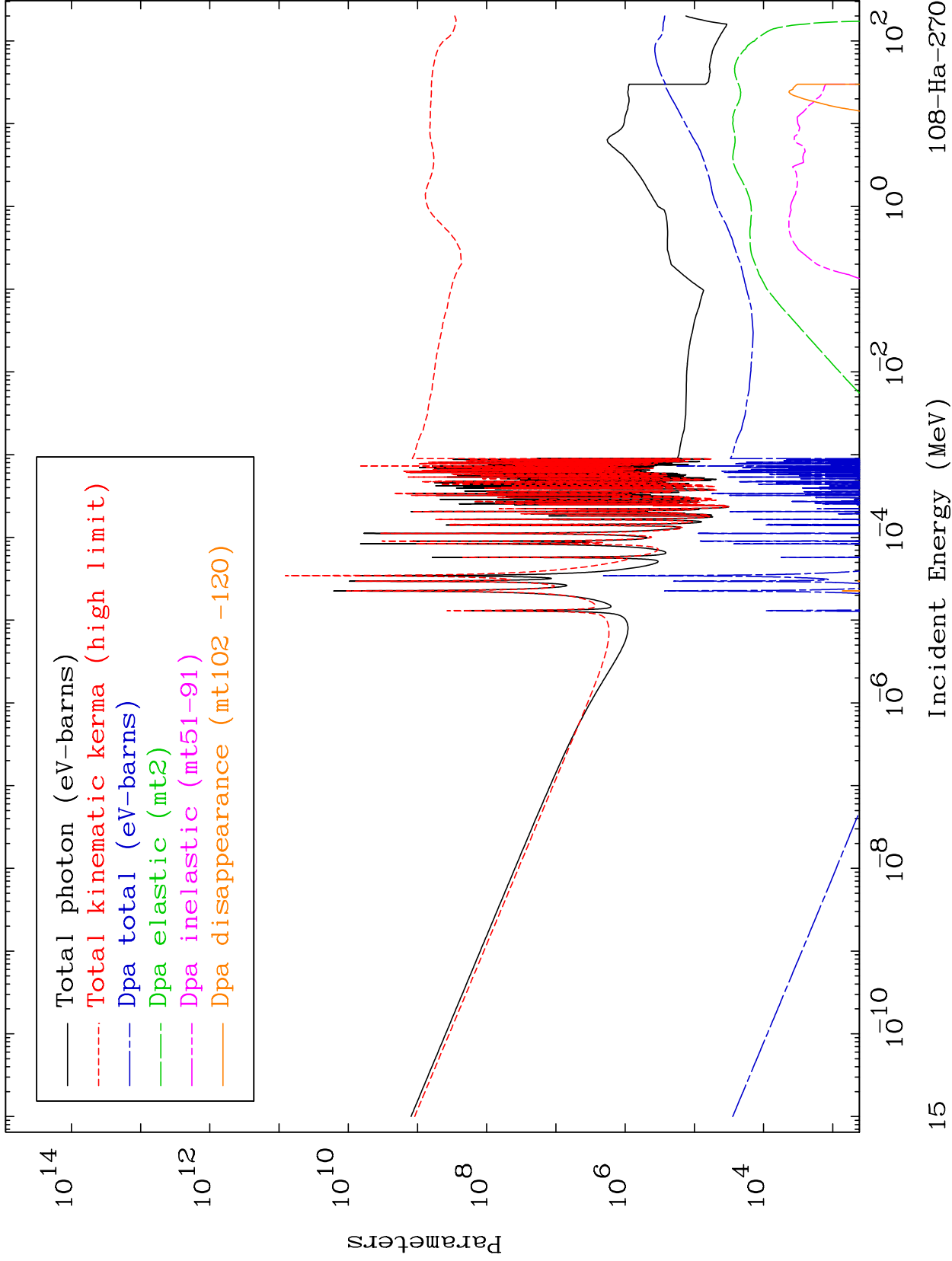
(n,He3) Levels  
293 Kelvin Cross Sections

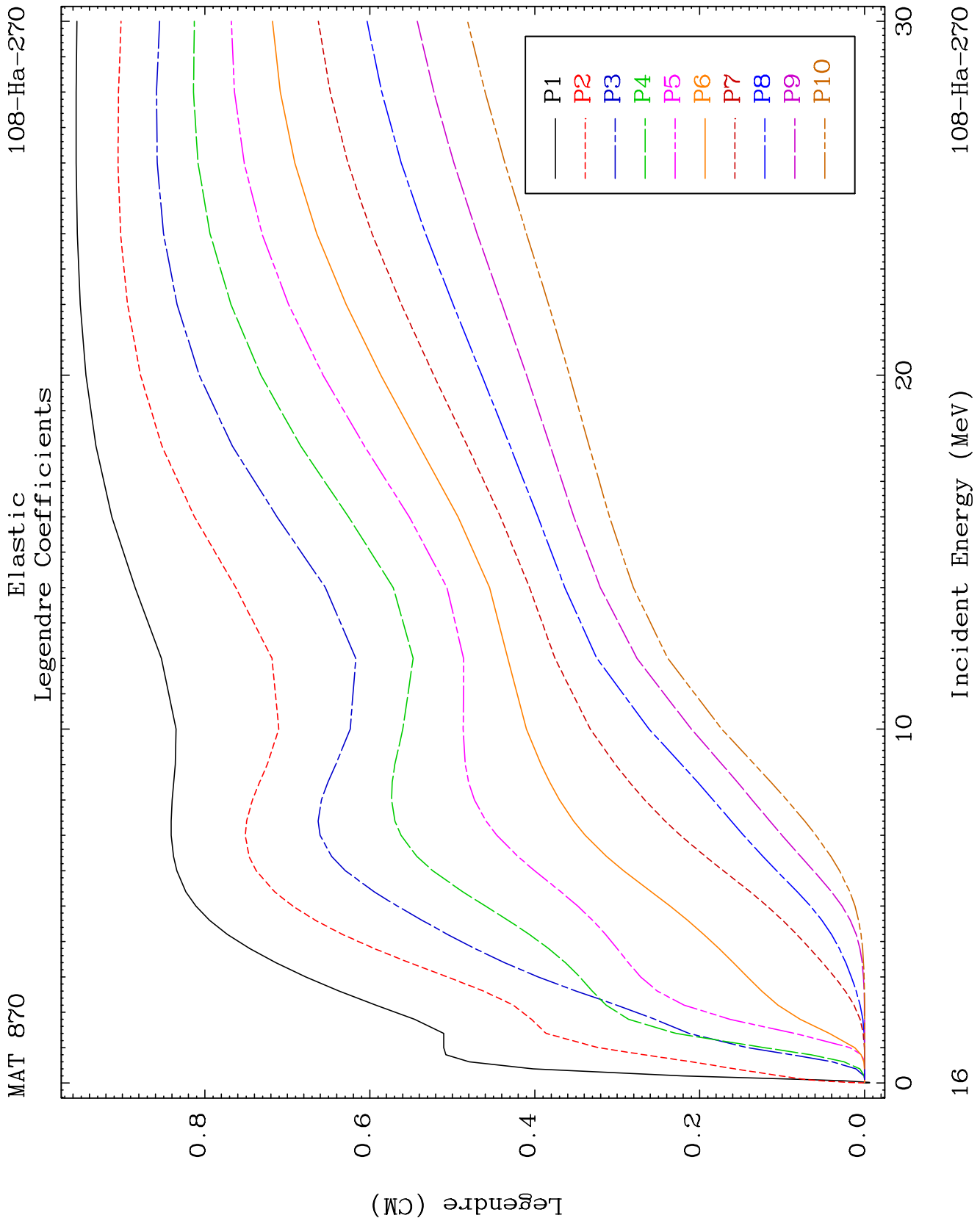
108-Ha-270

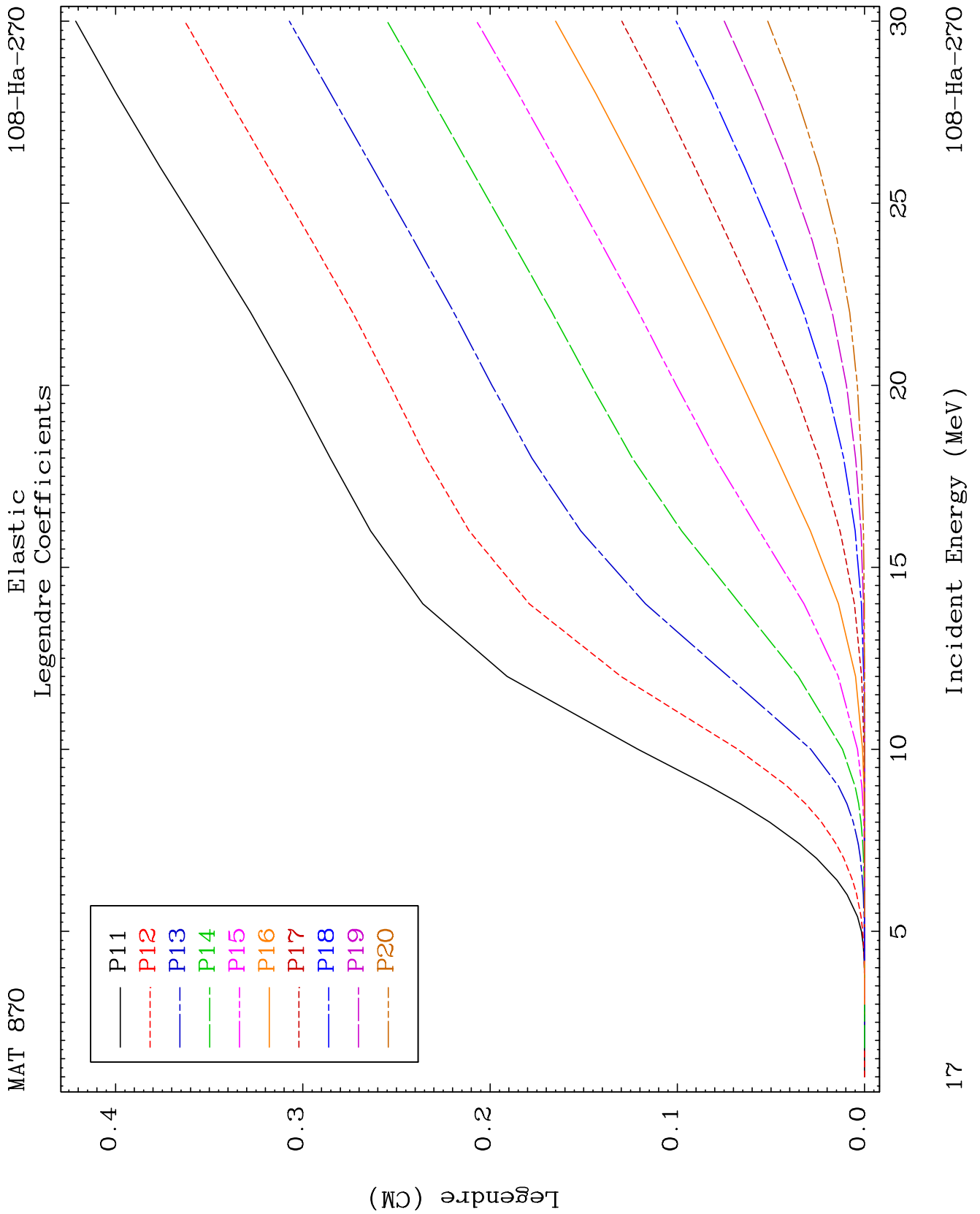


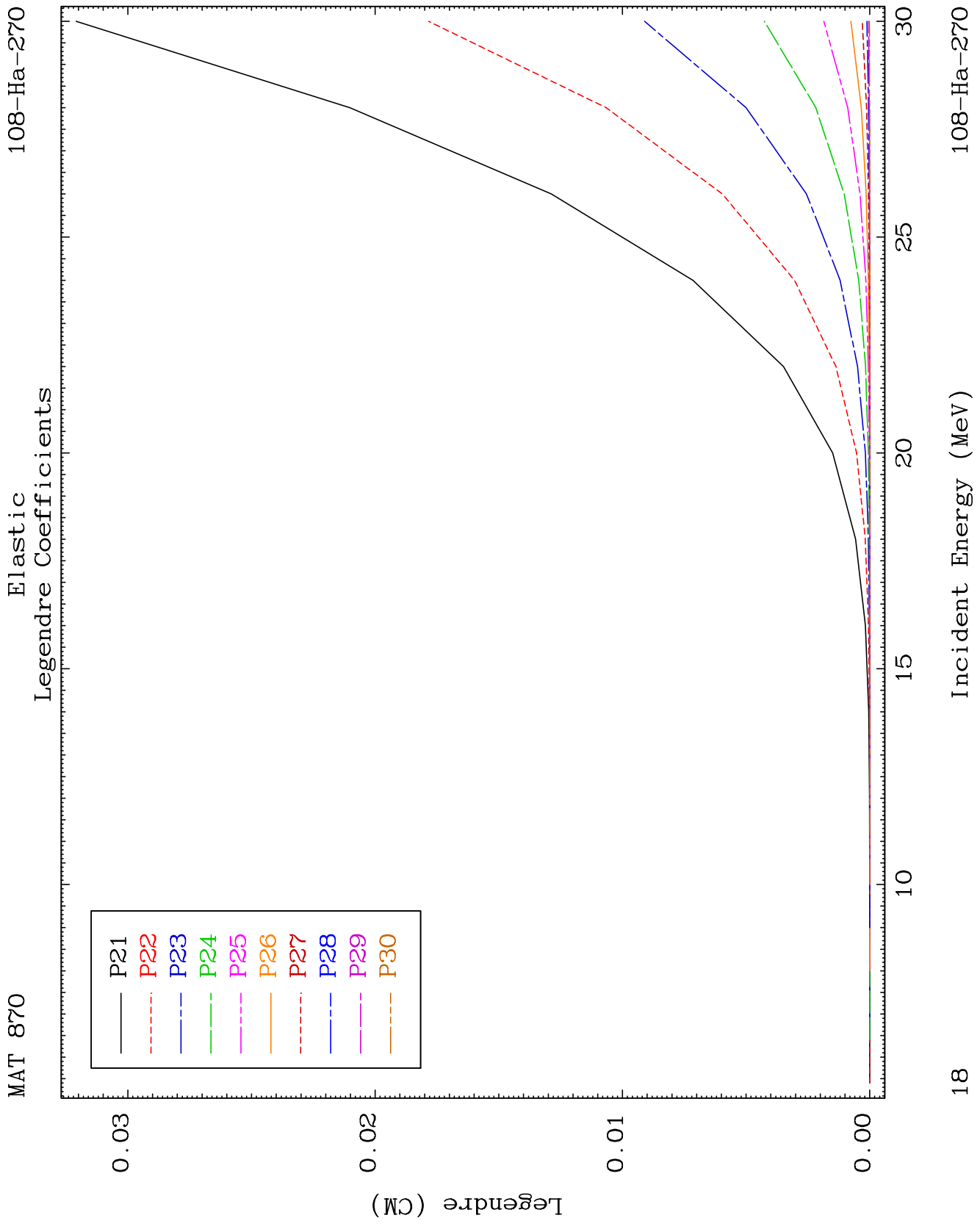










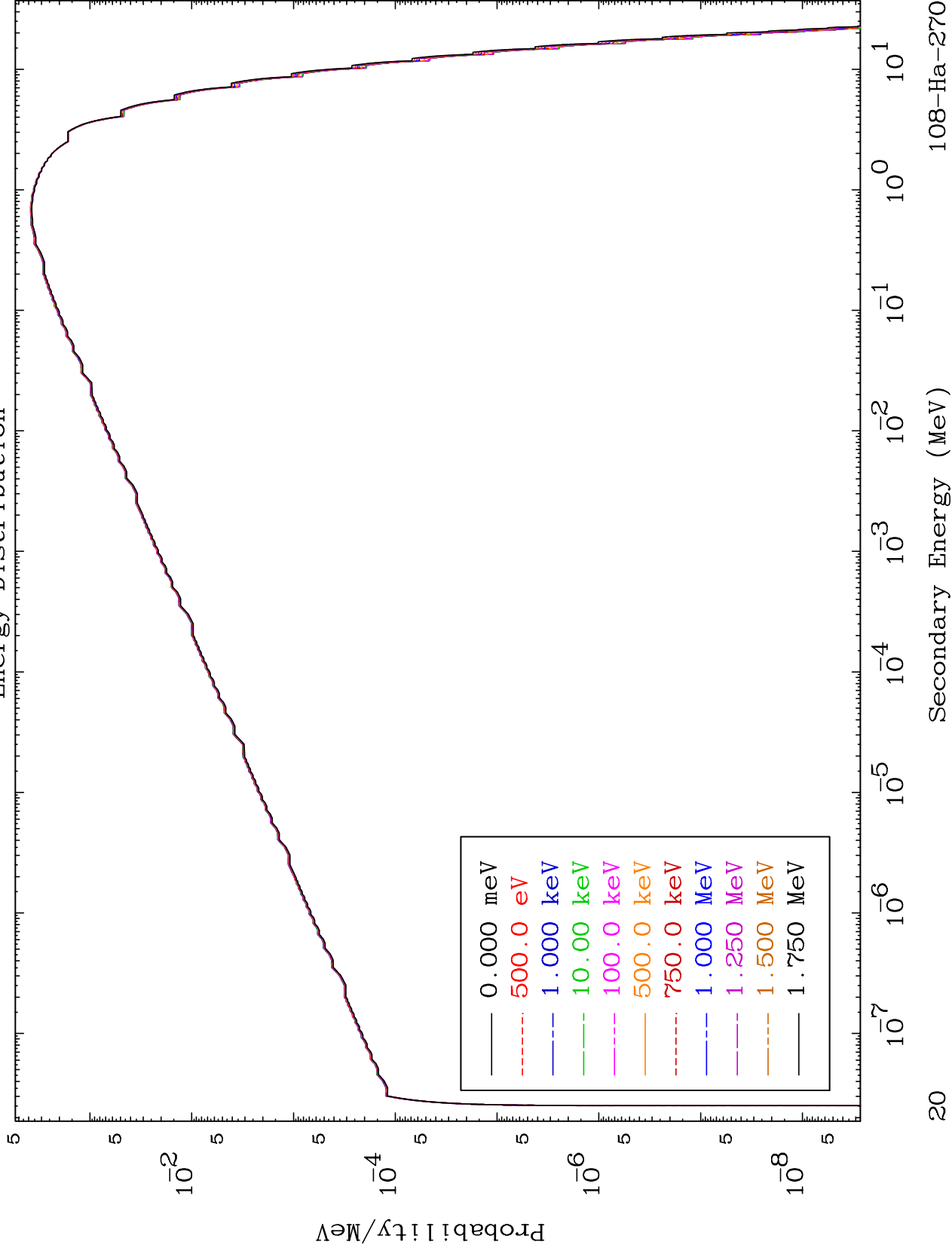


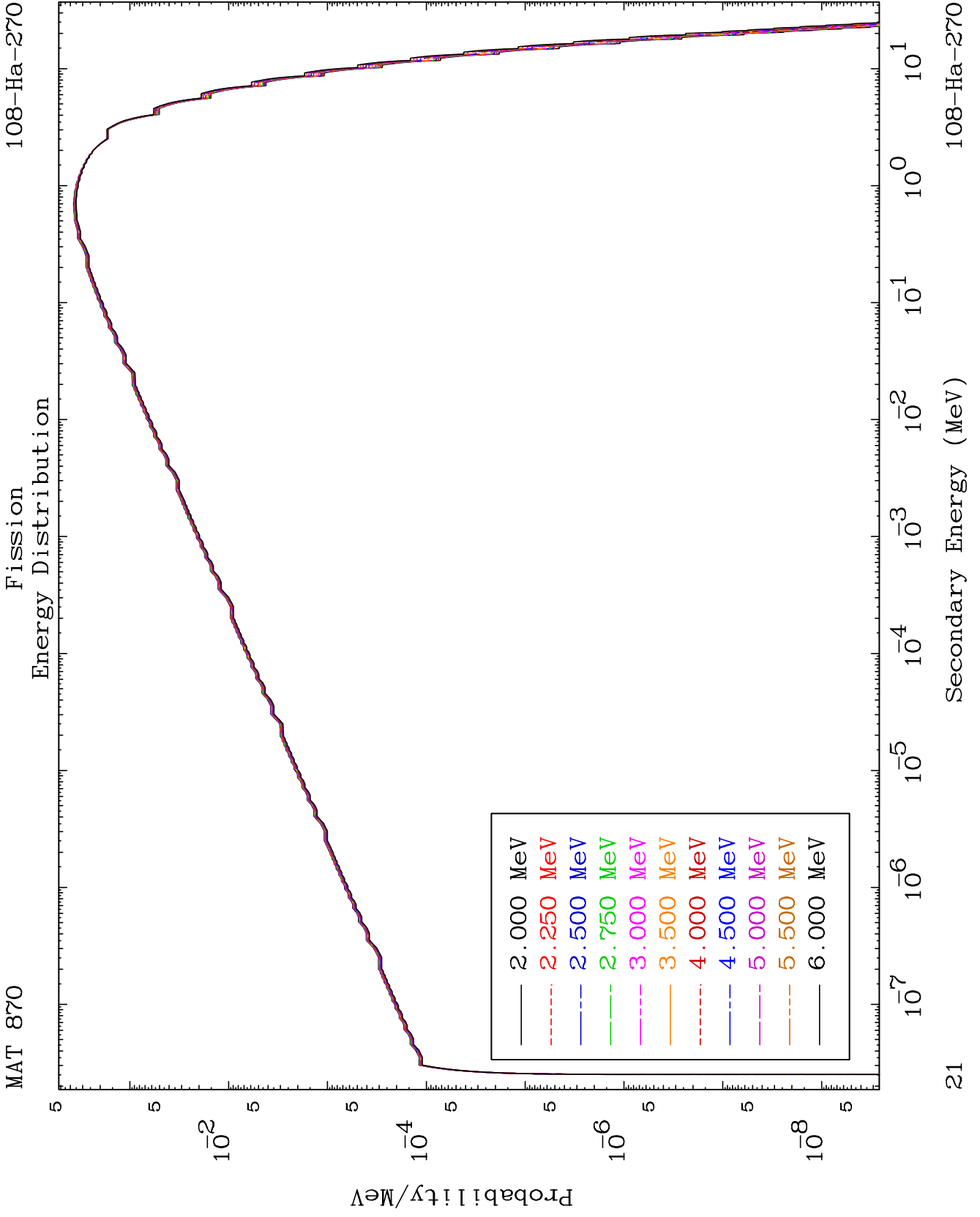


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# Fission Energy Distribution

108-Ha-270

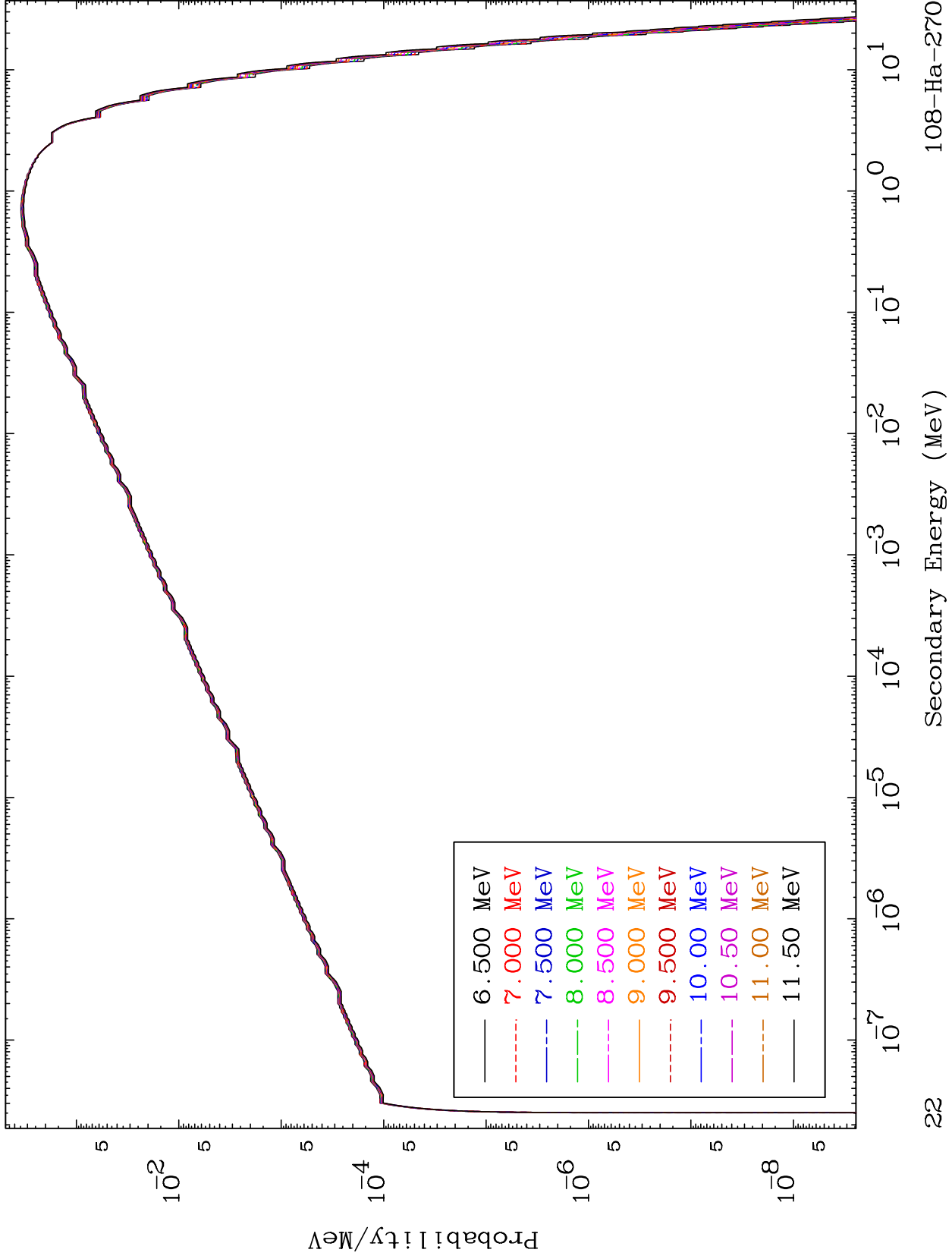




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### Fission Energy Distribution

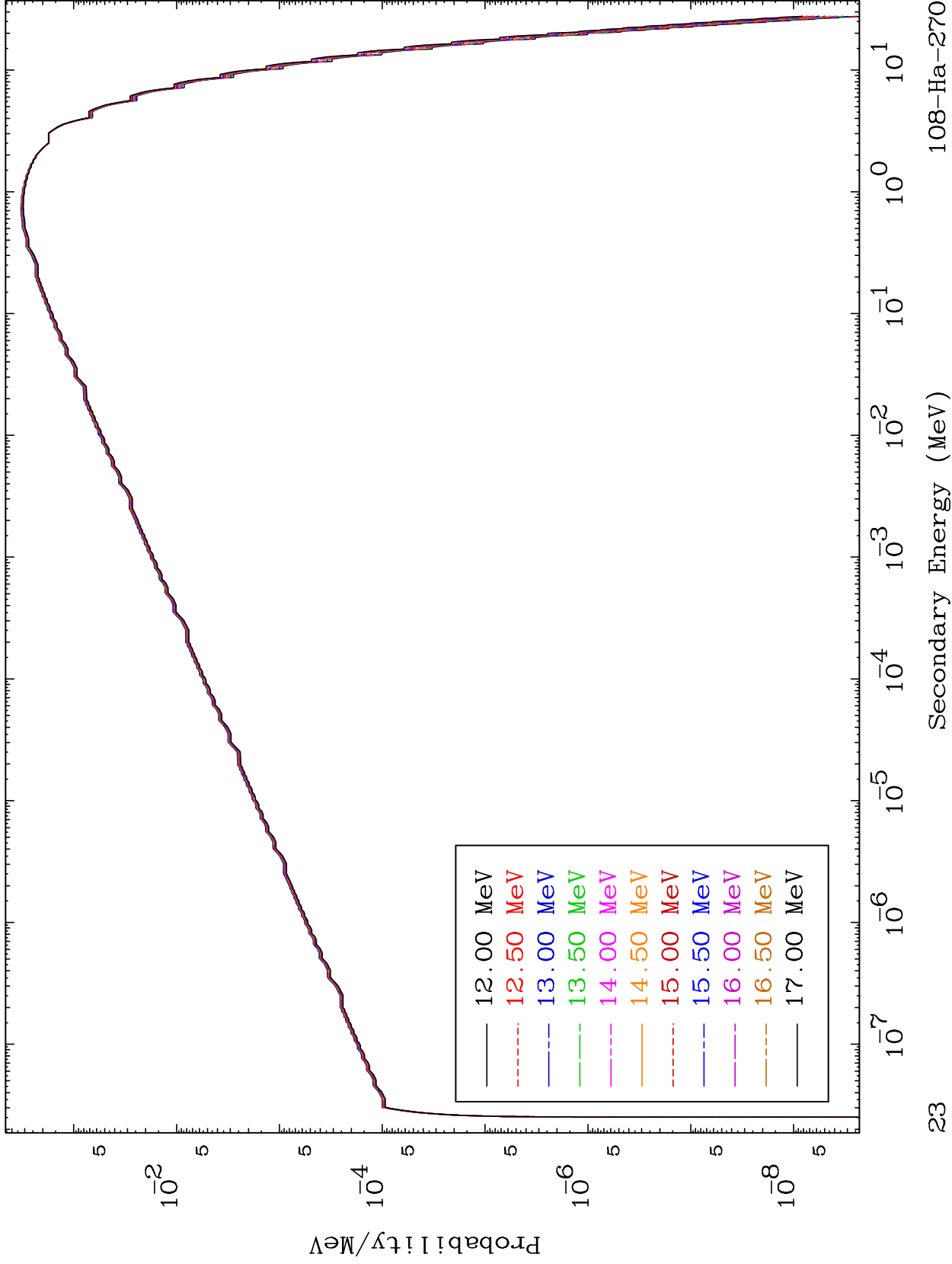
108-Ha-270



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### Fission Energy Distribution

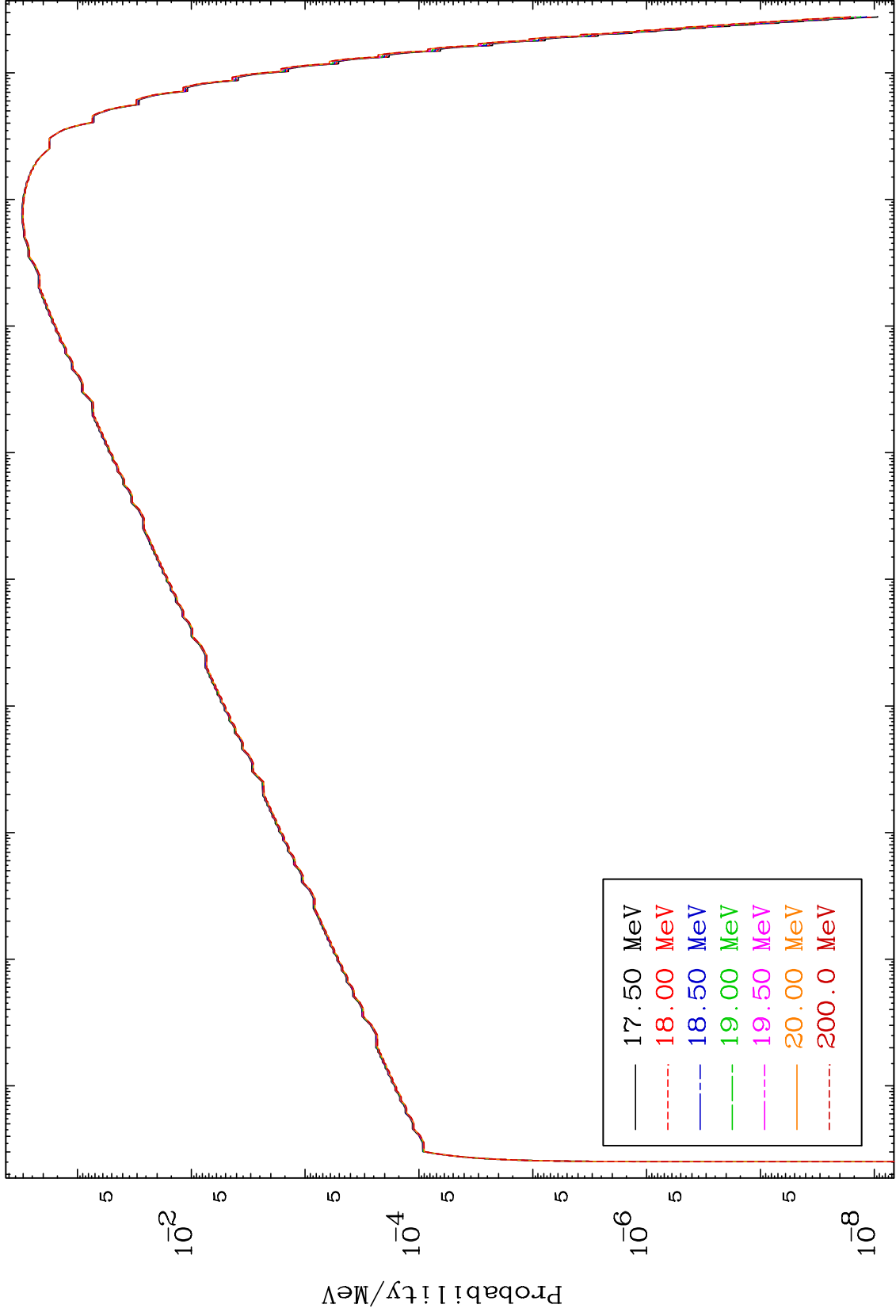
108-Ha-270



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### Fission Energy Distribution

108-Ha-270



MAT 870

$(n,2n) \alpha$

108-Ha-270

Radionuclide Production Cross Section

