

Program EVALPLOT
(Version 2017-1)

by

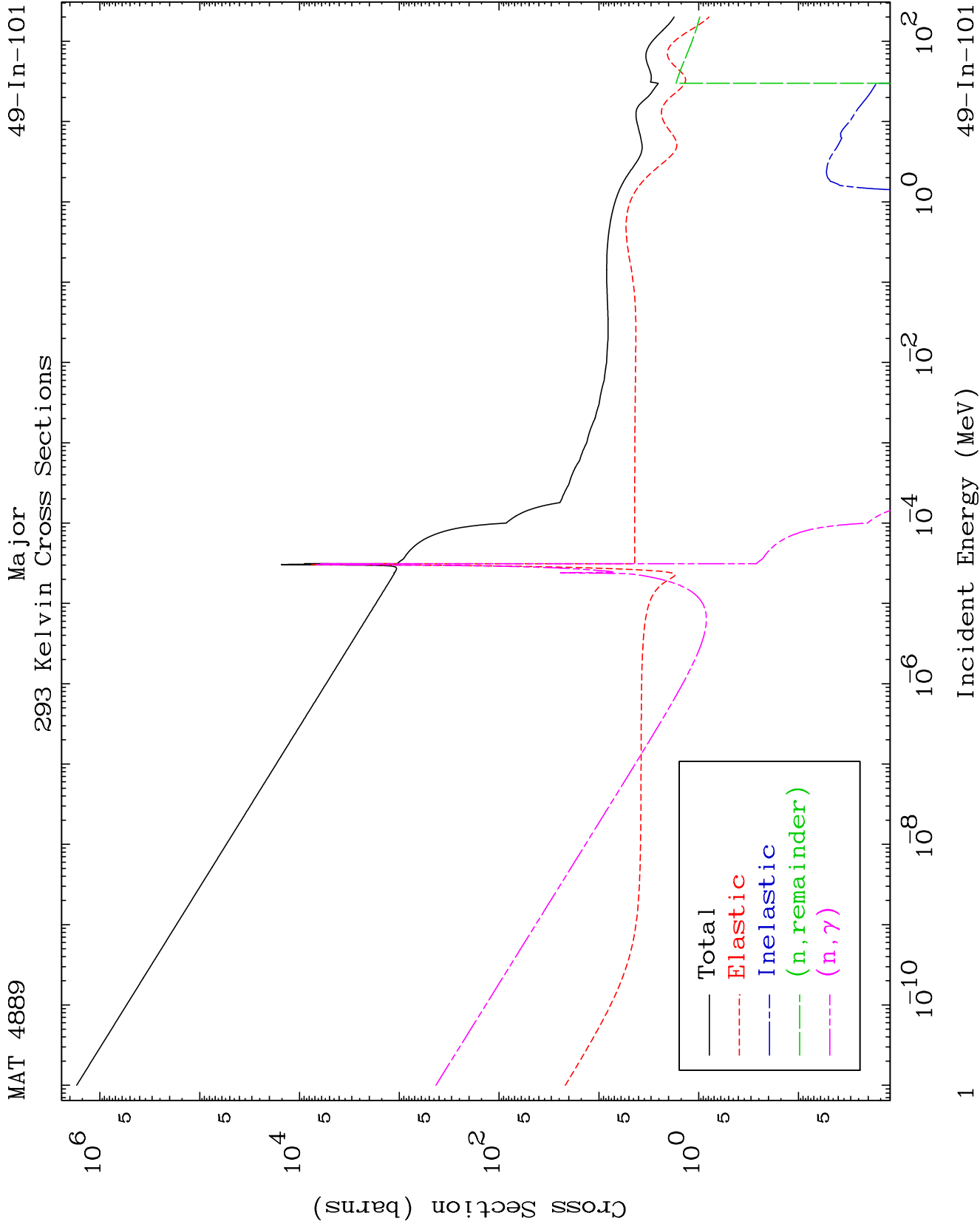
Dermott E. Cullen
(Present Contact Information)

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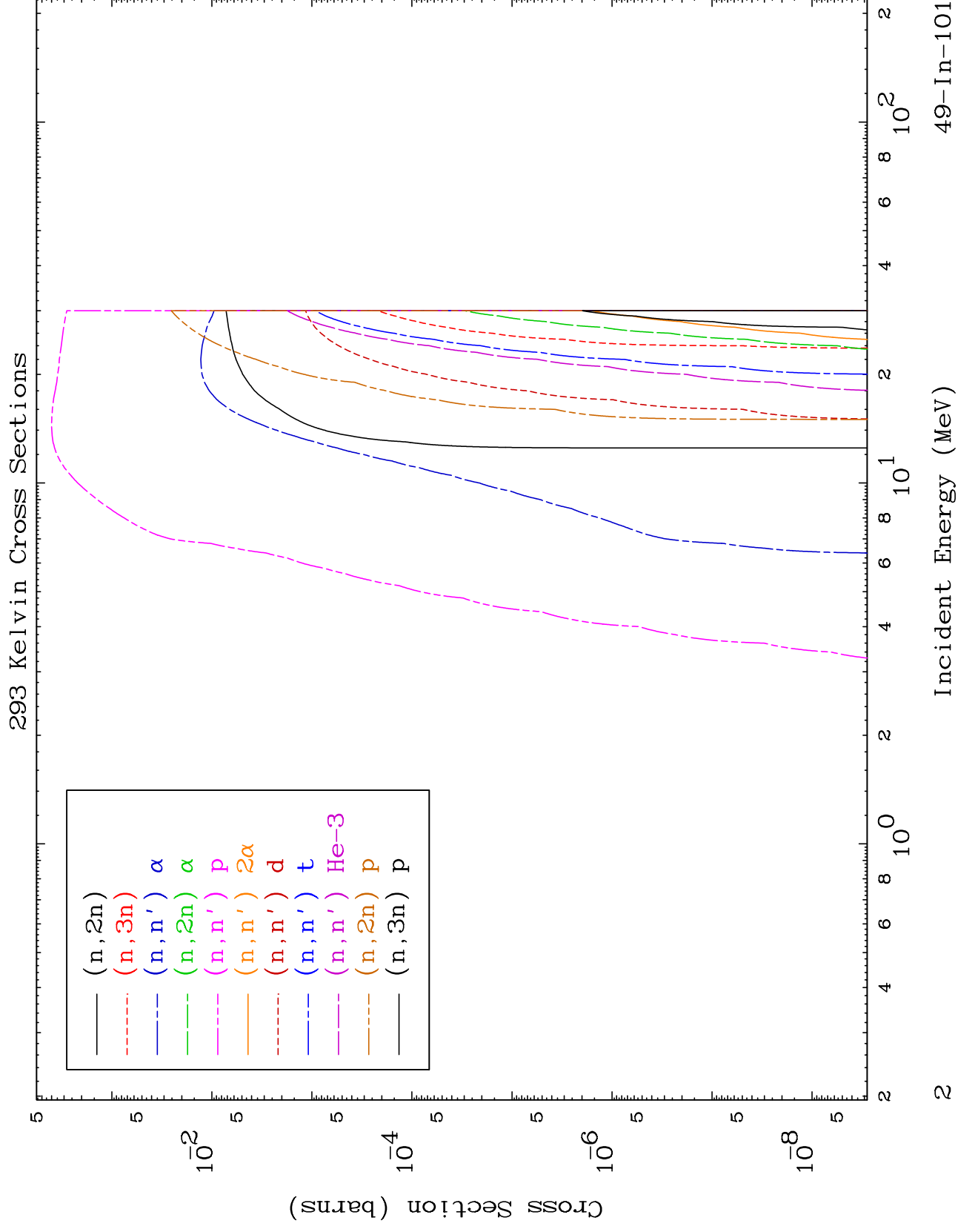
Press Mouse Button to Start

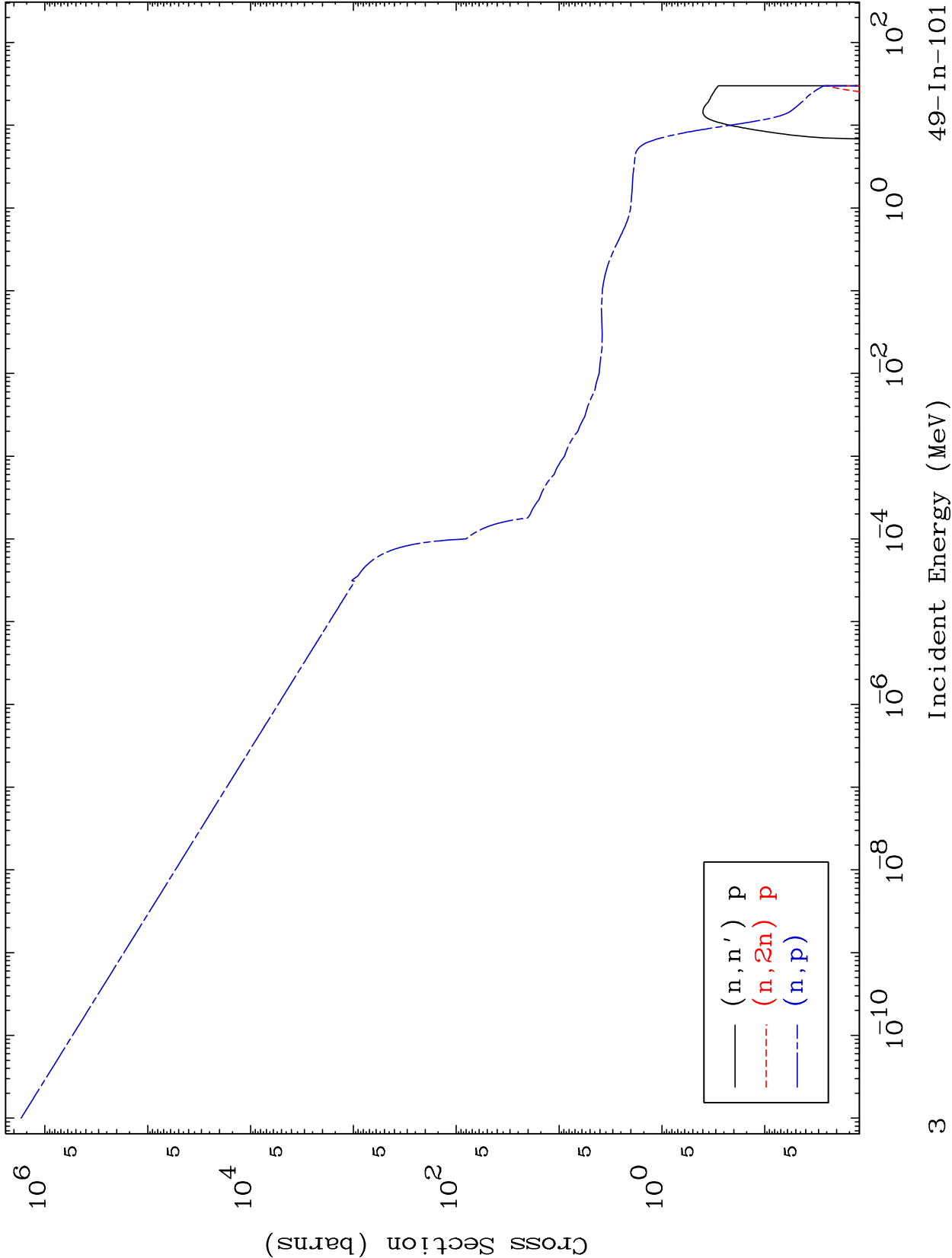


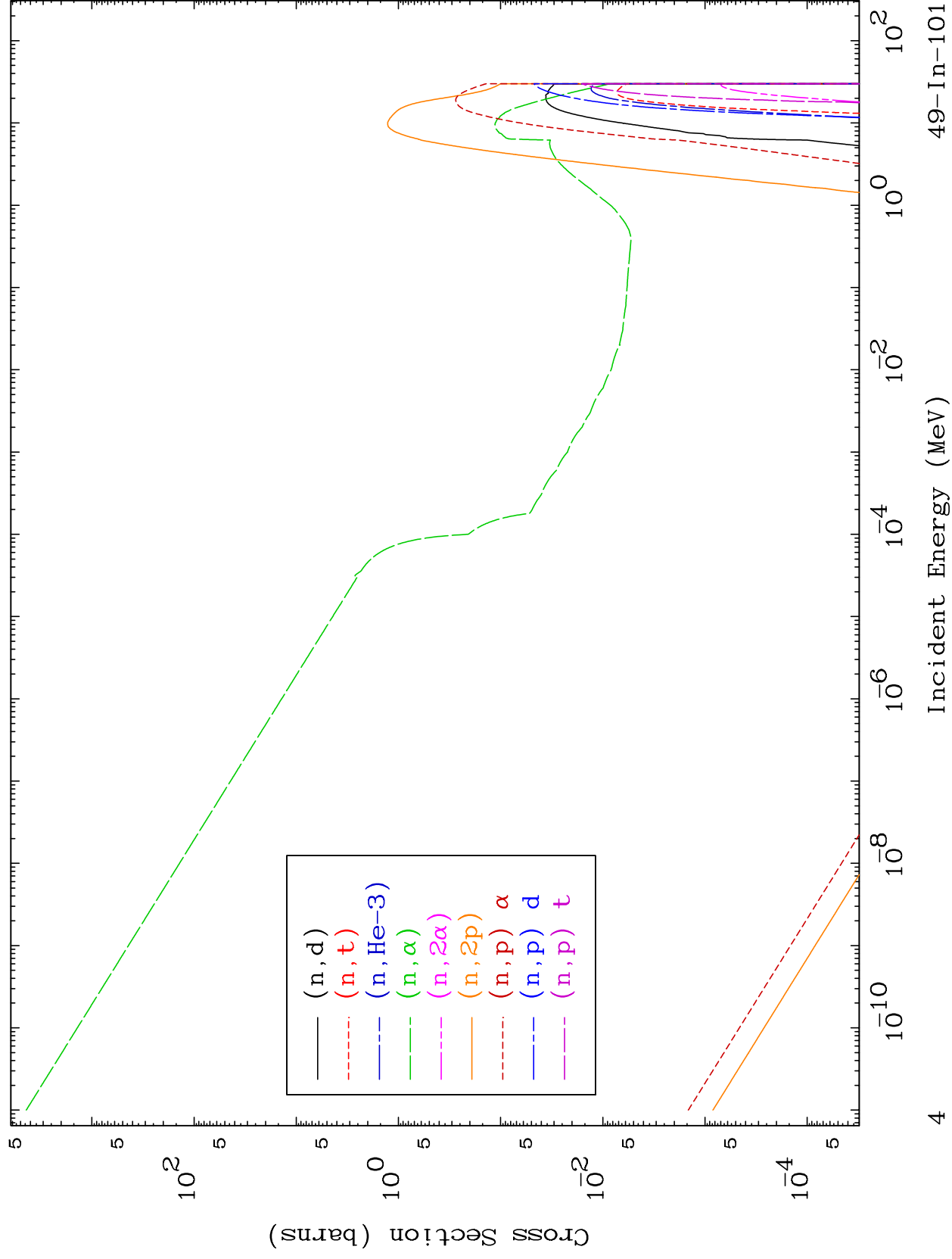
MAT 4889

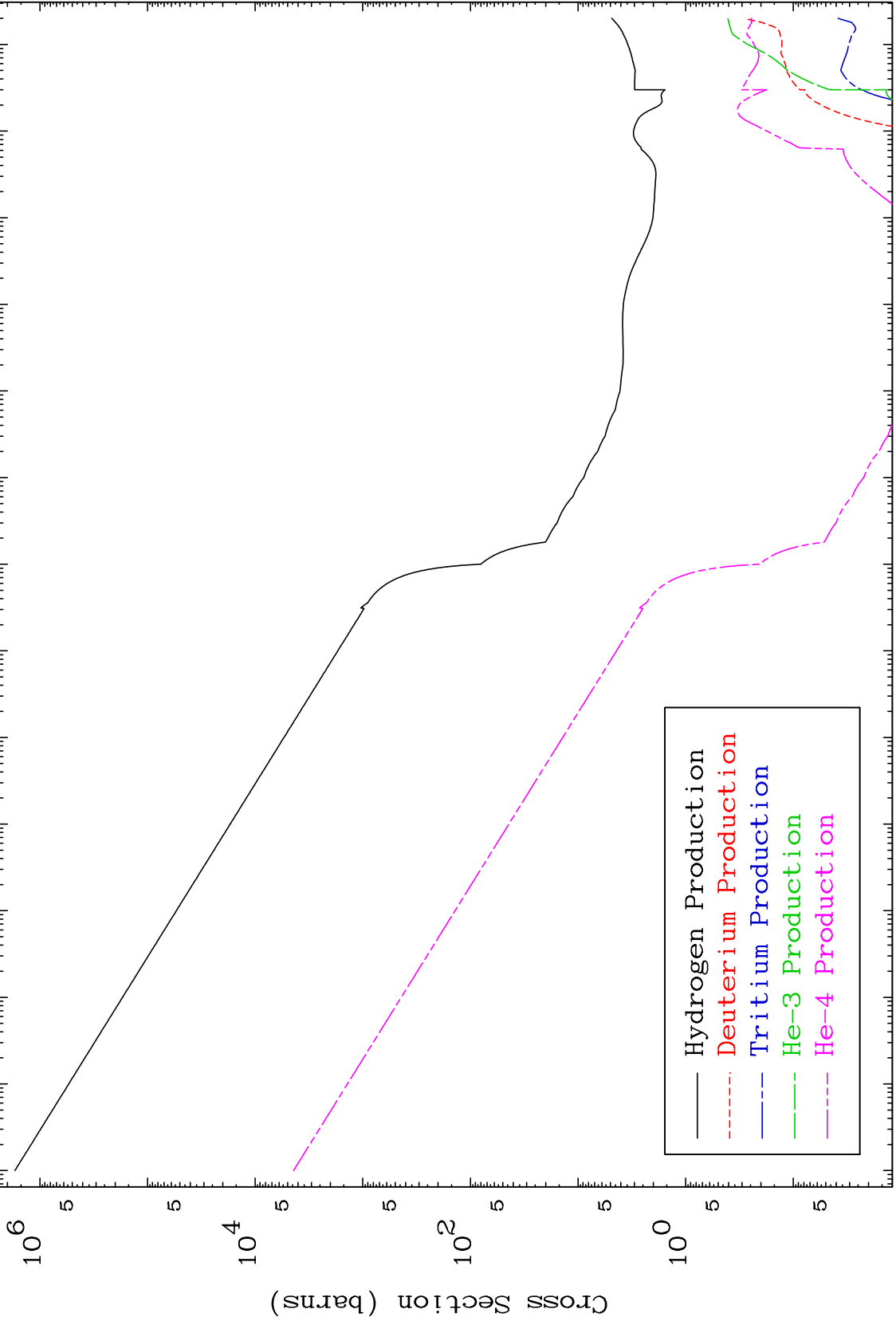
Neutron Production

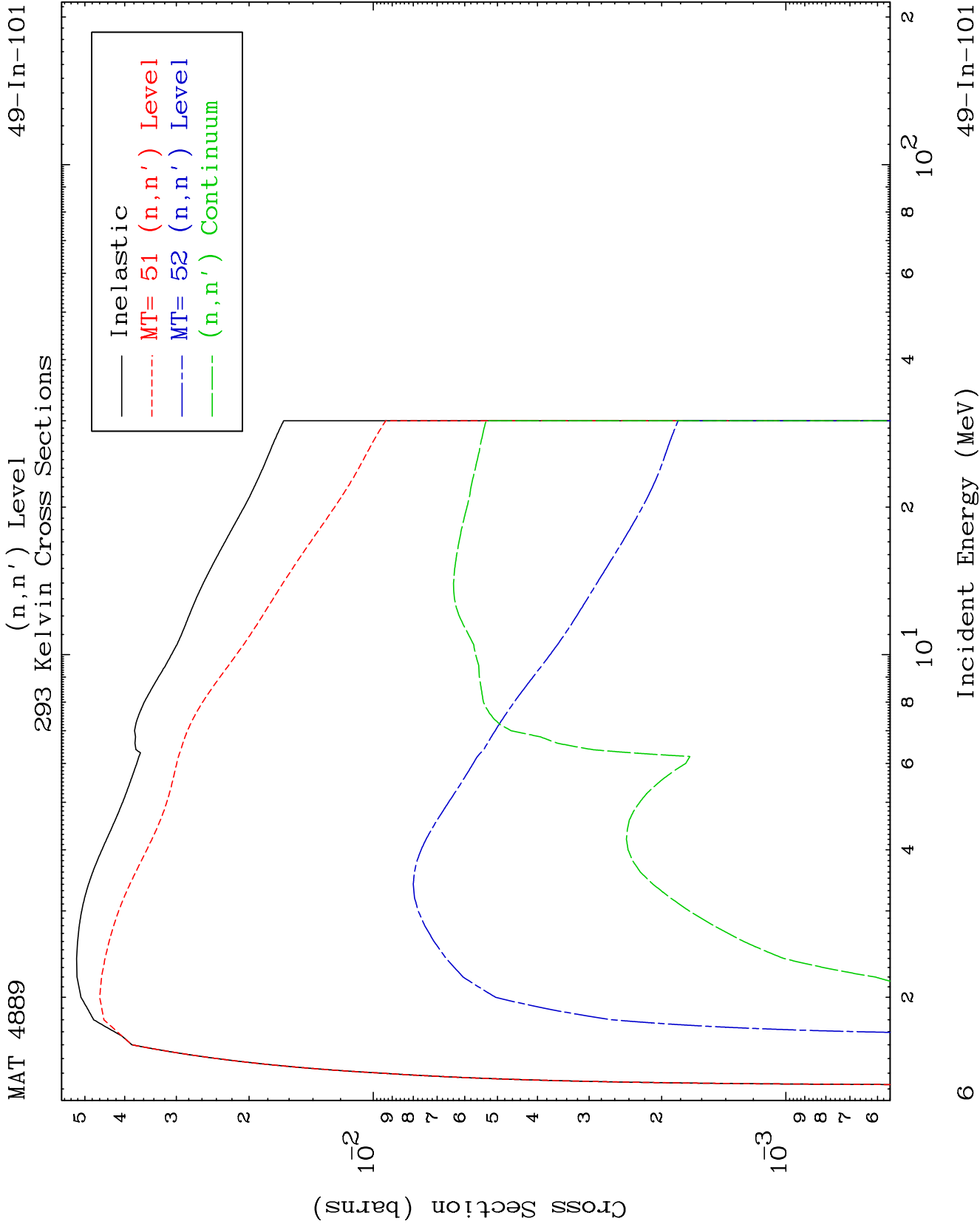
49-In-101

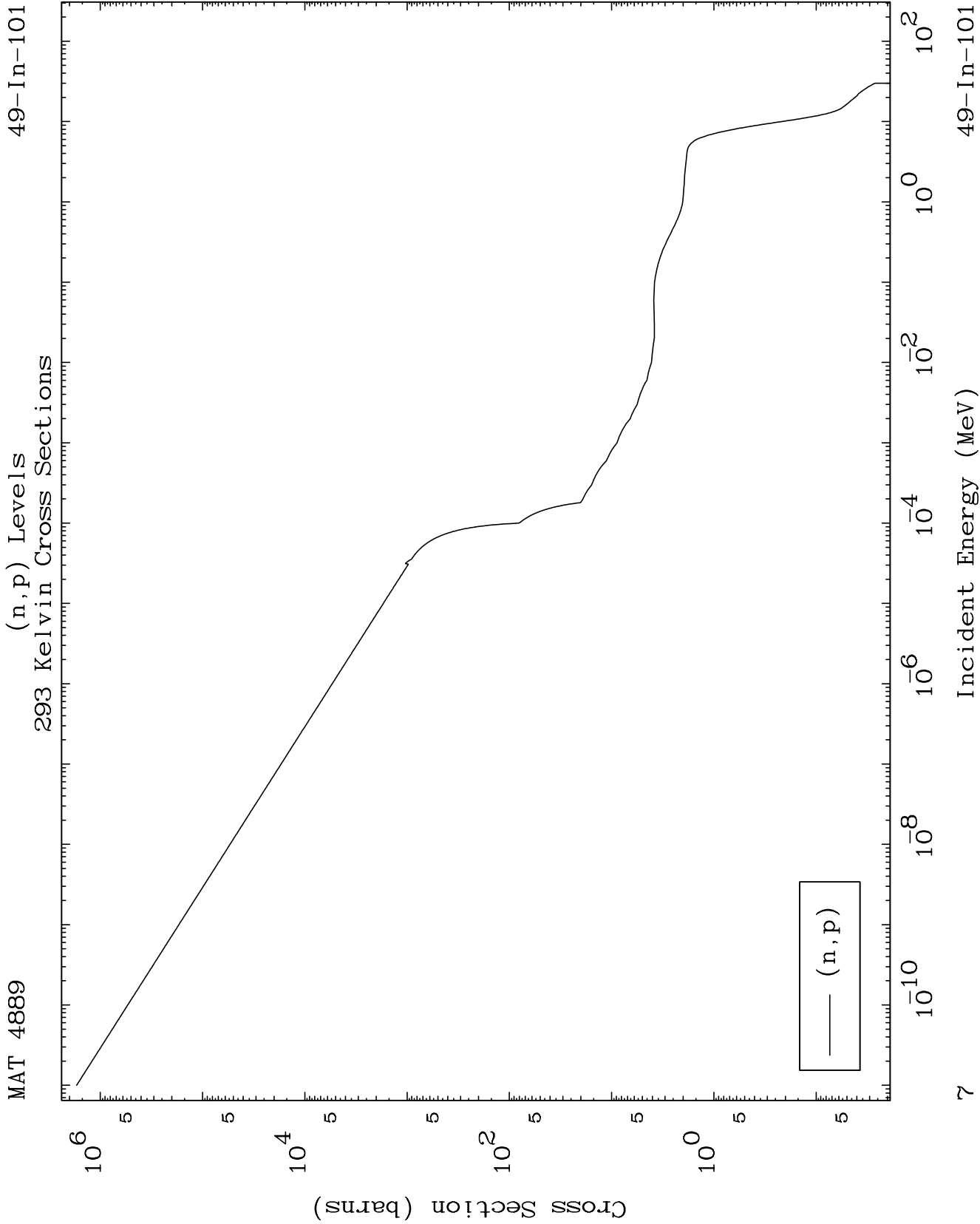








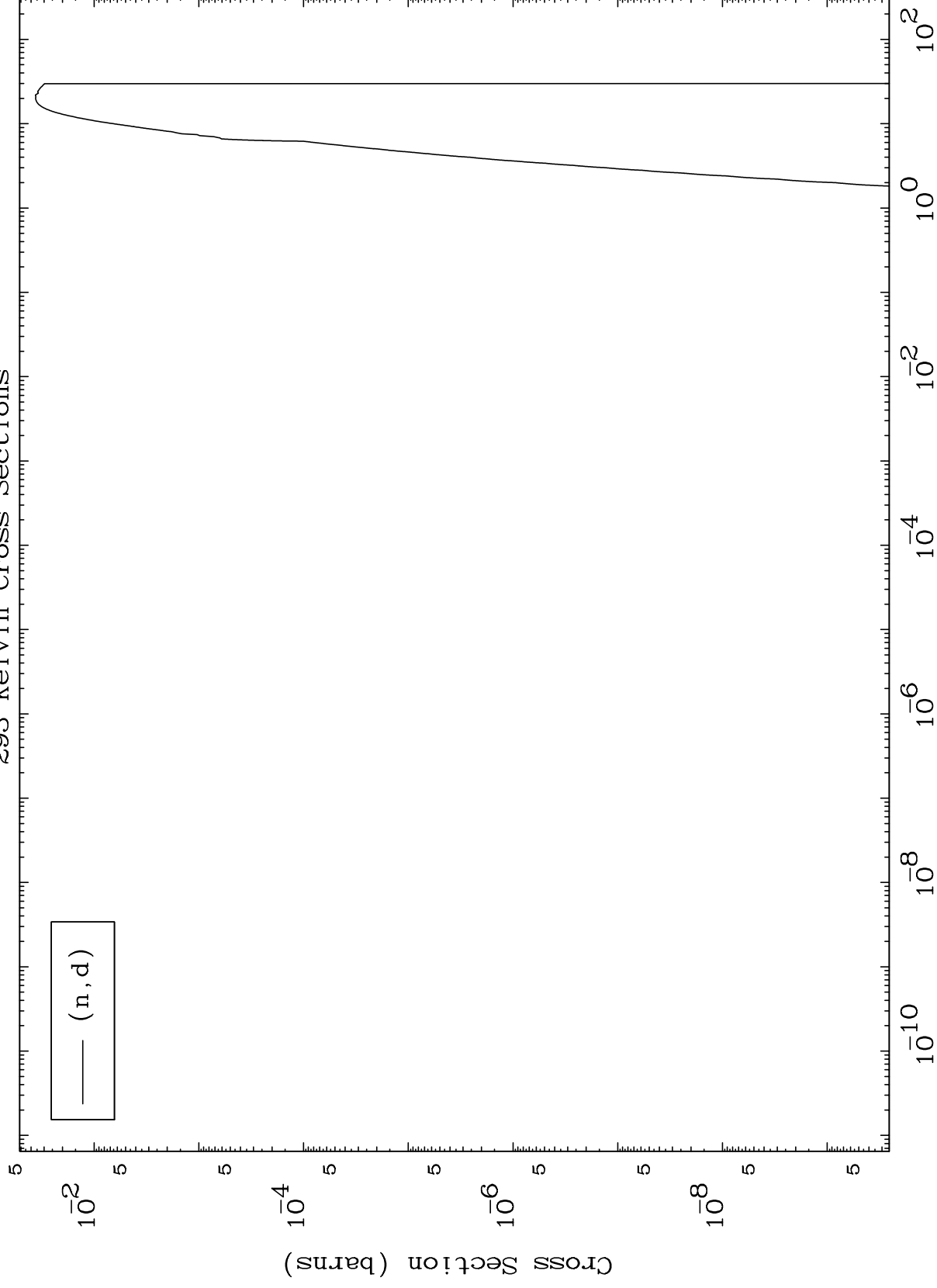


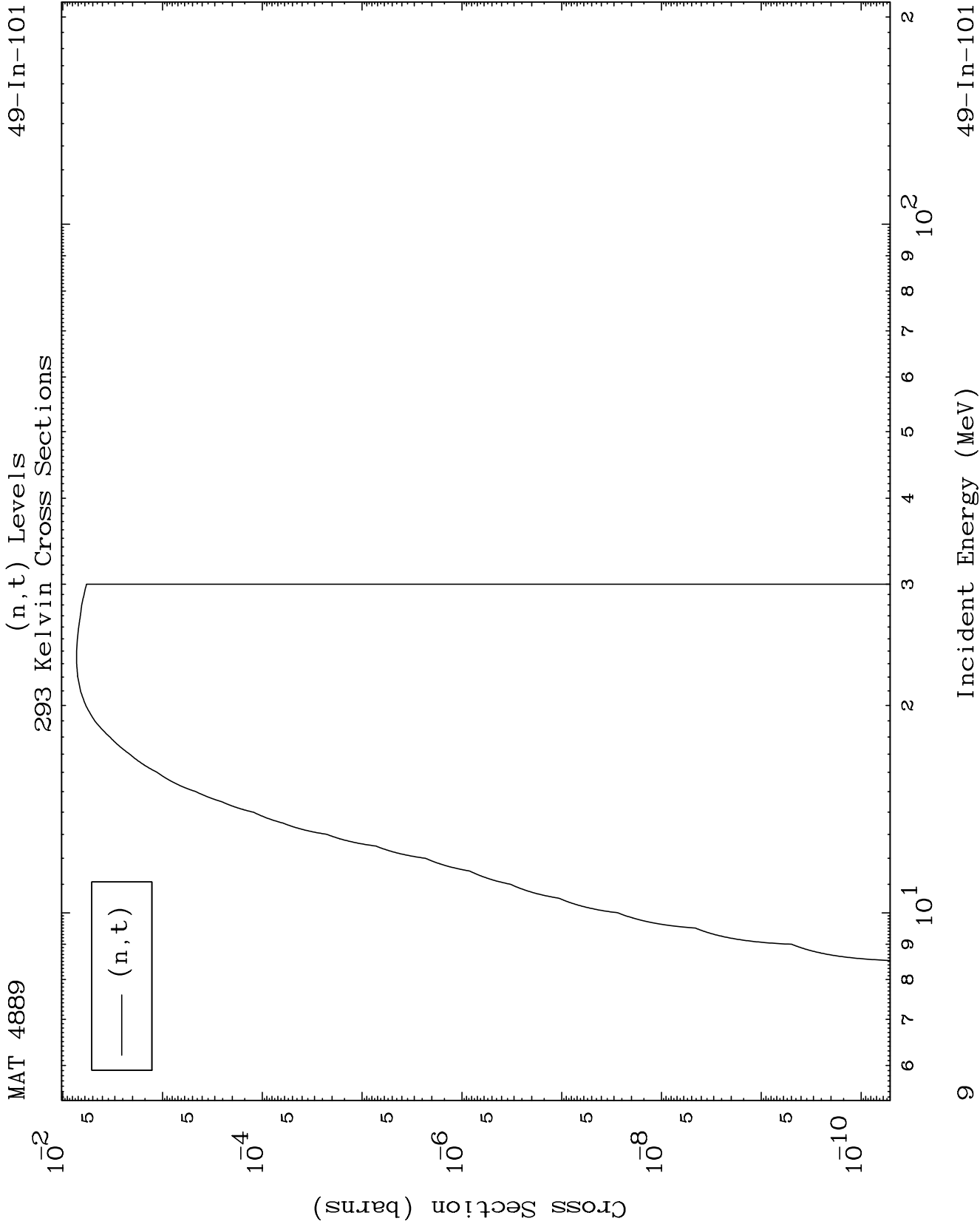


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(n,d) Levels
293 Kelvin Cross Sections

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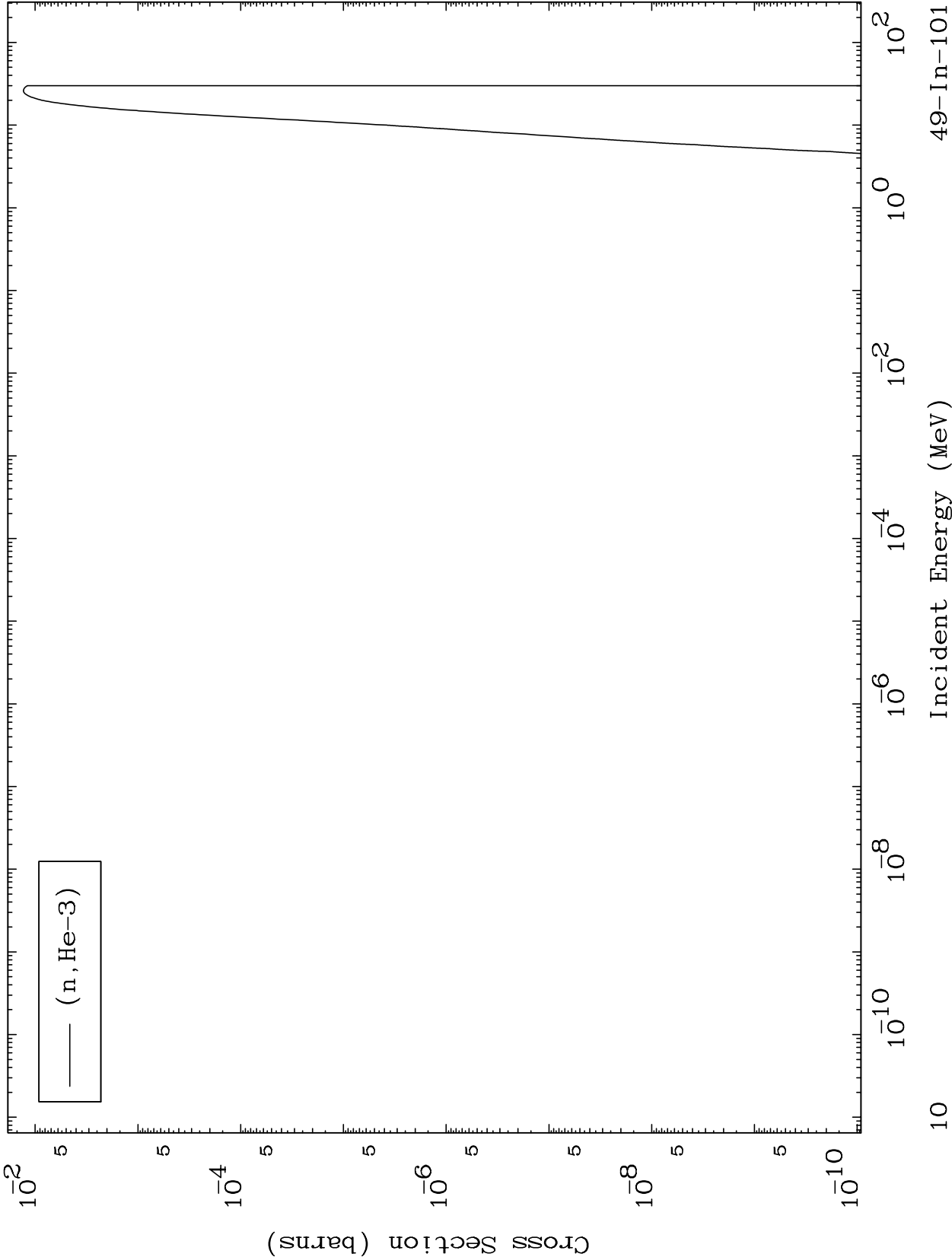




MAT 4889

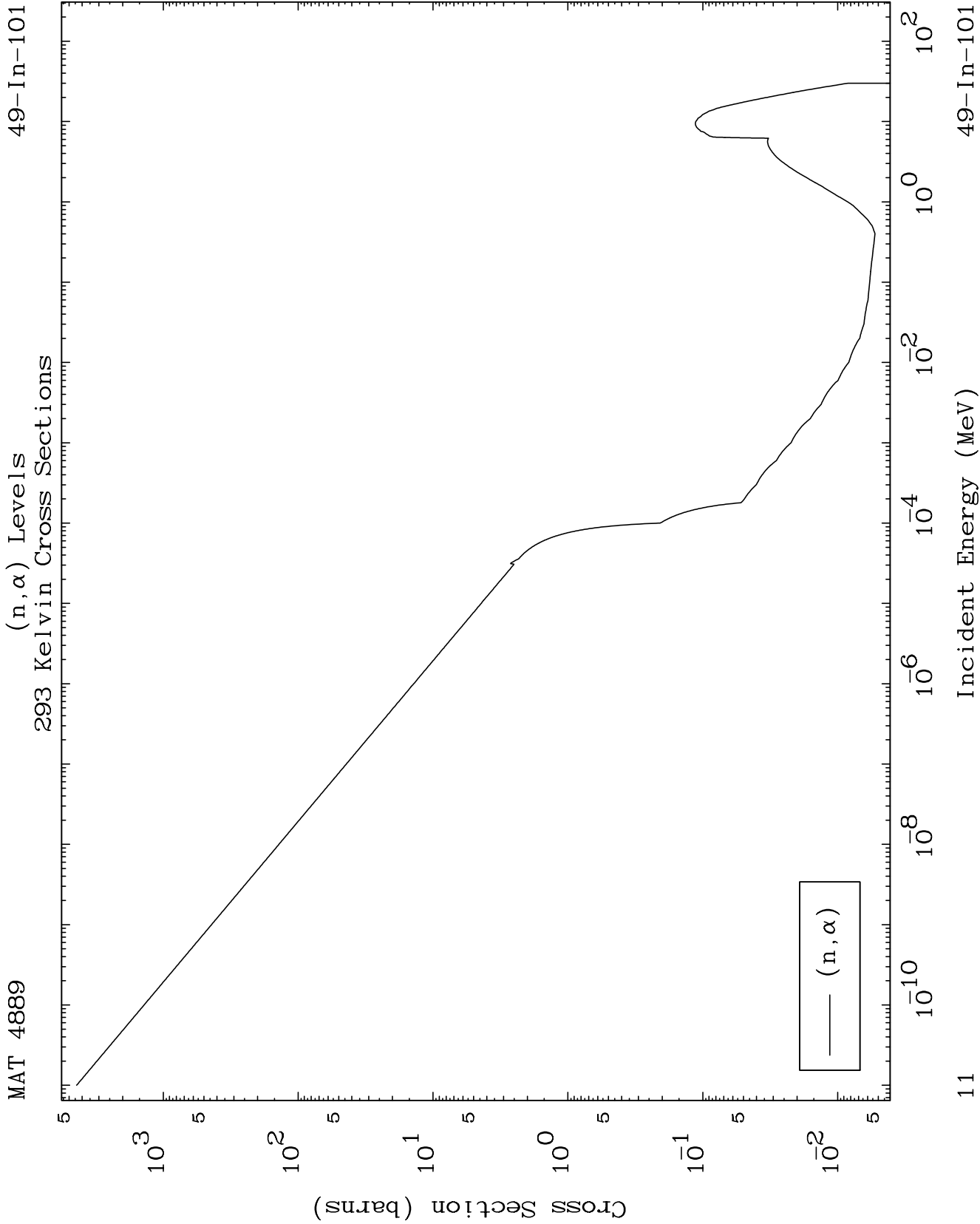
(n,He3) Levels
293 Kelvin Cross Sections

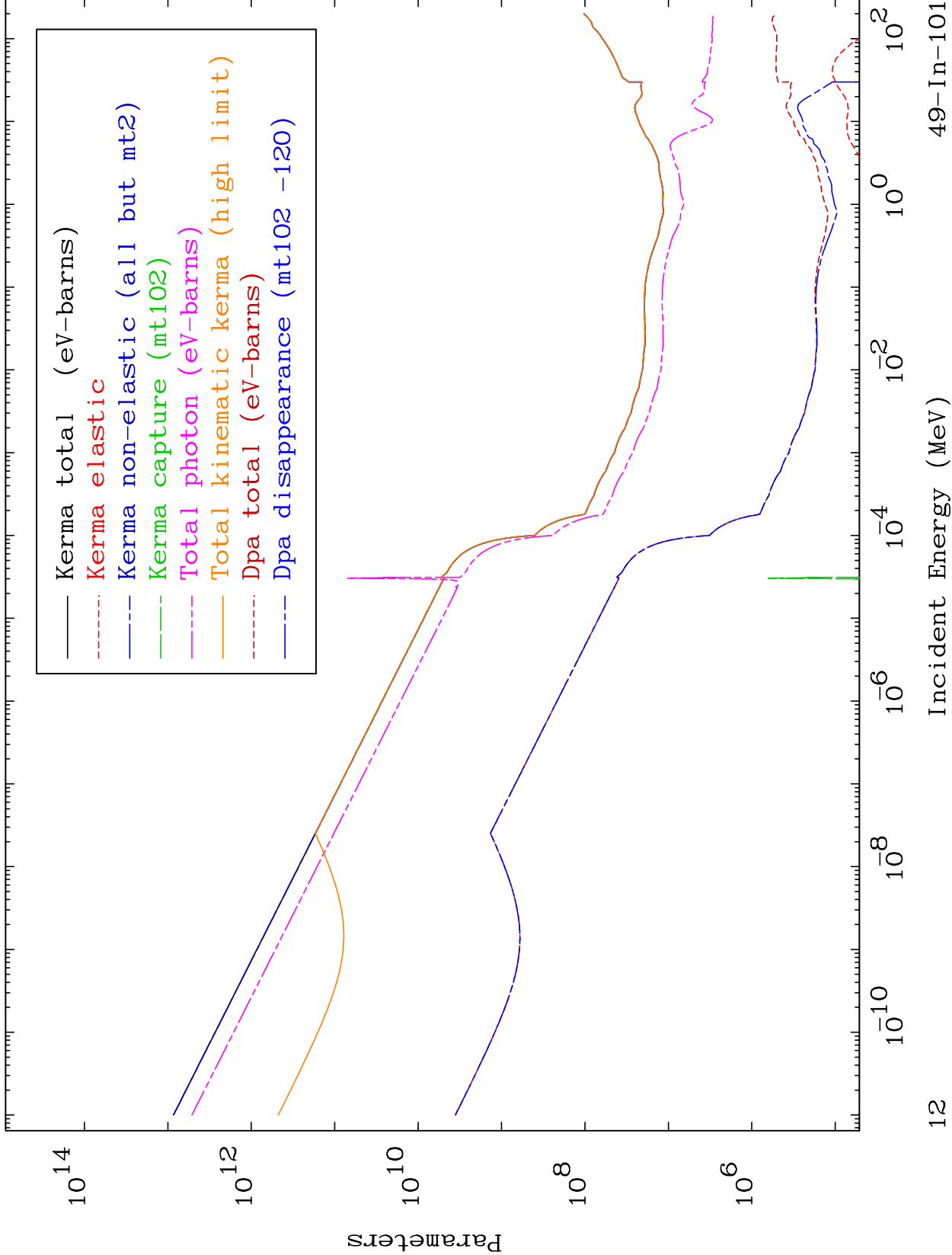
49-In-101

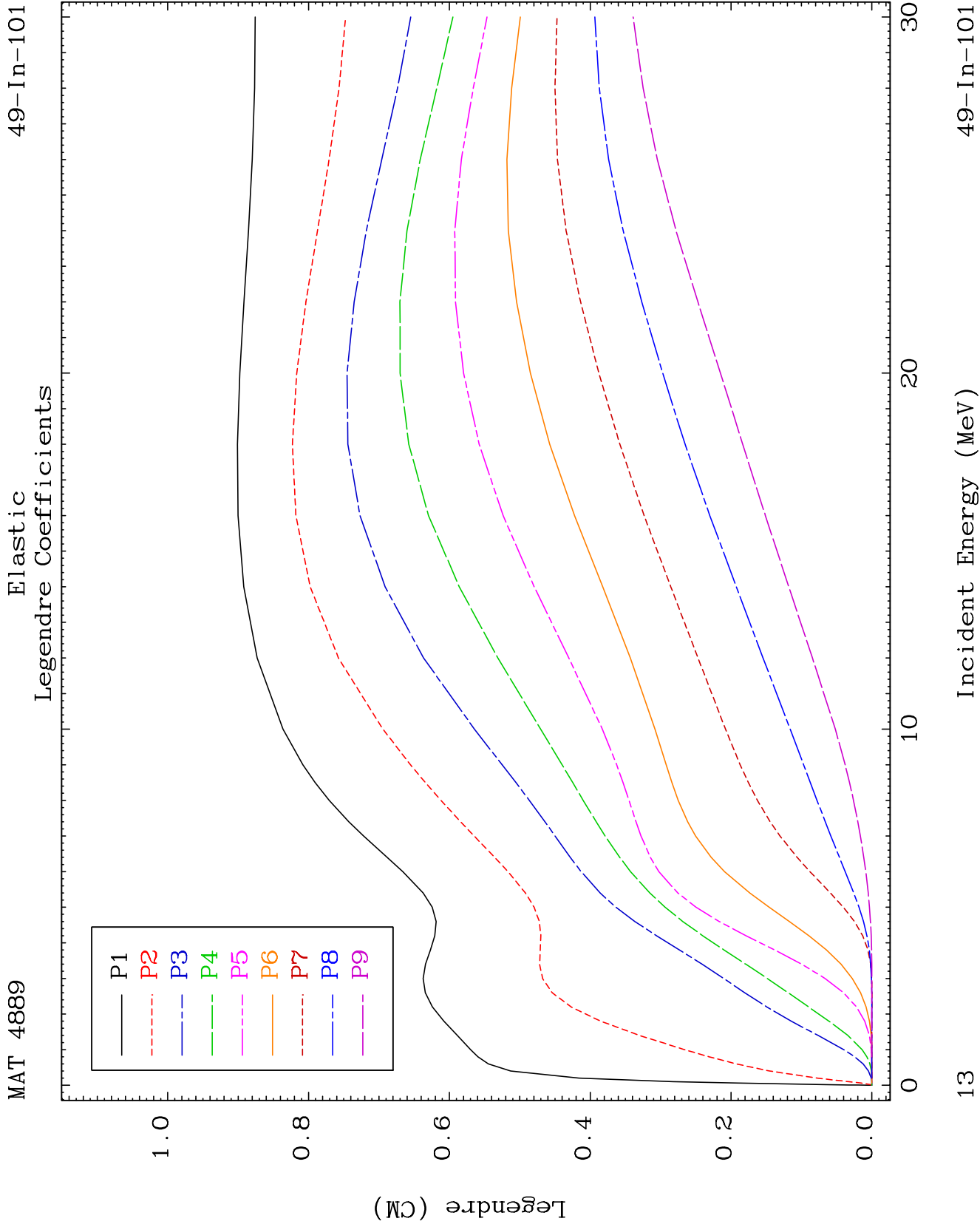


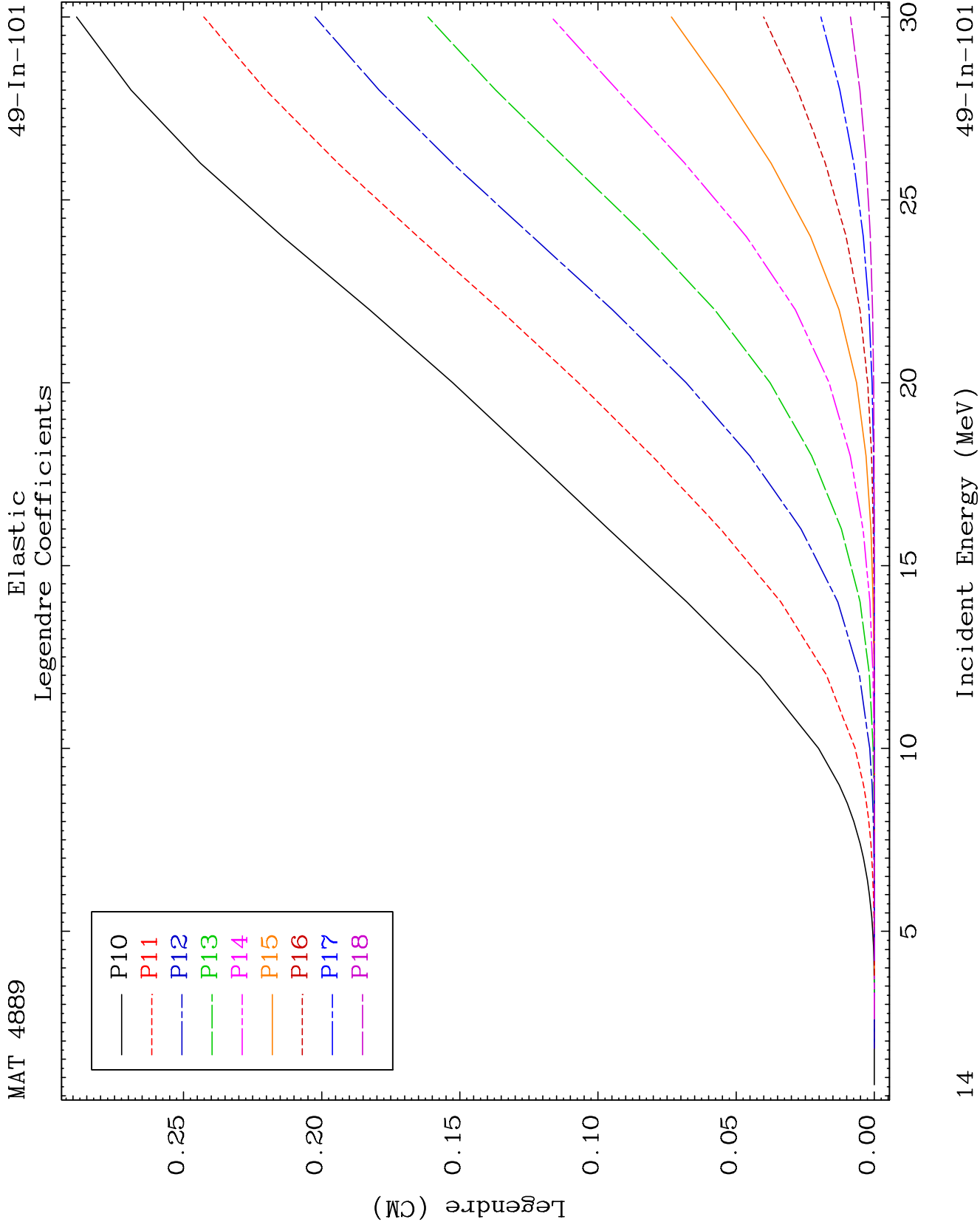
Incident Energy (MeV)

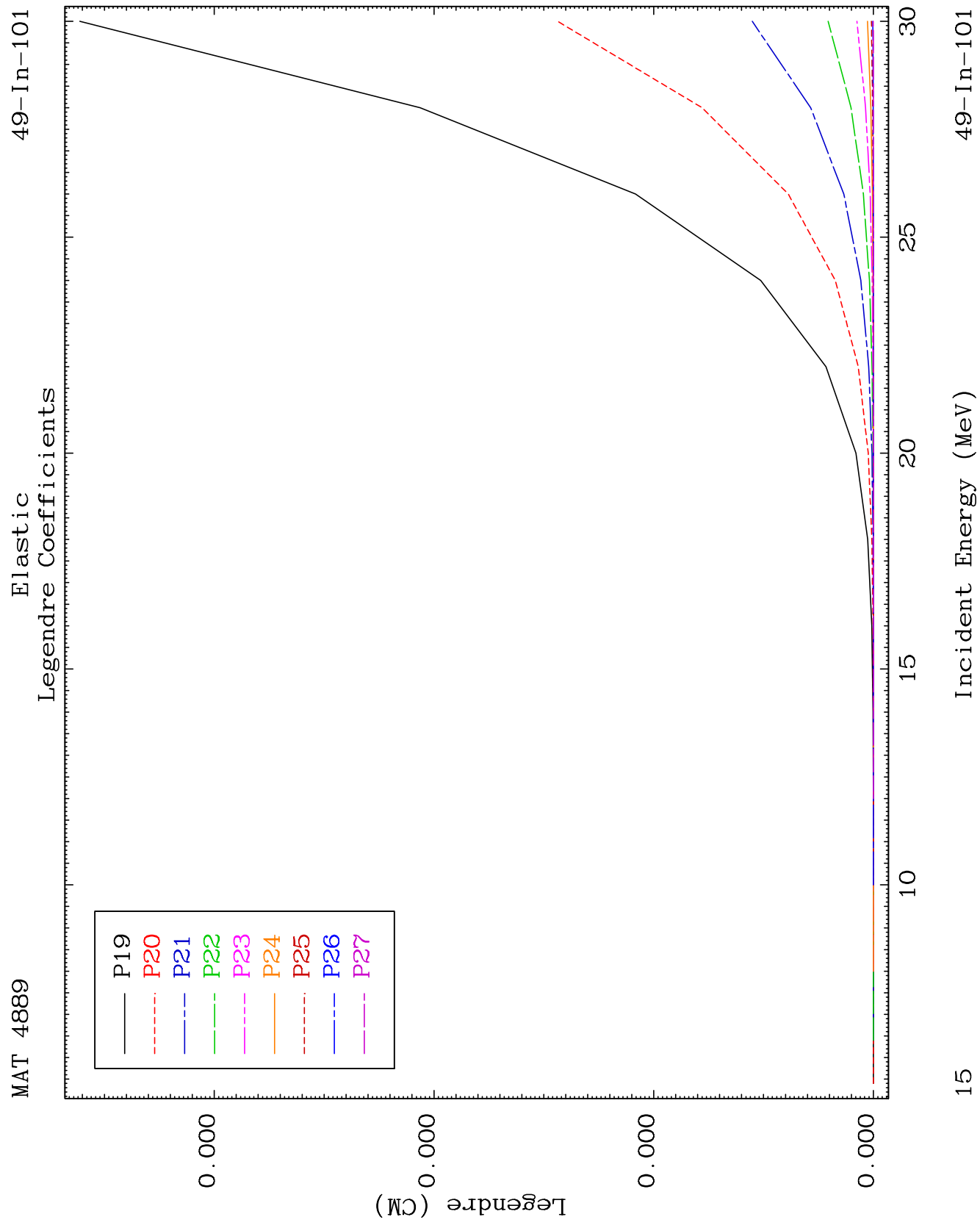
49-In-101

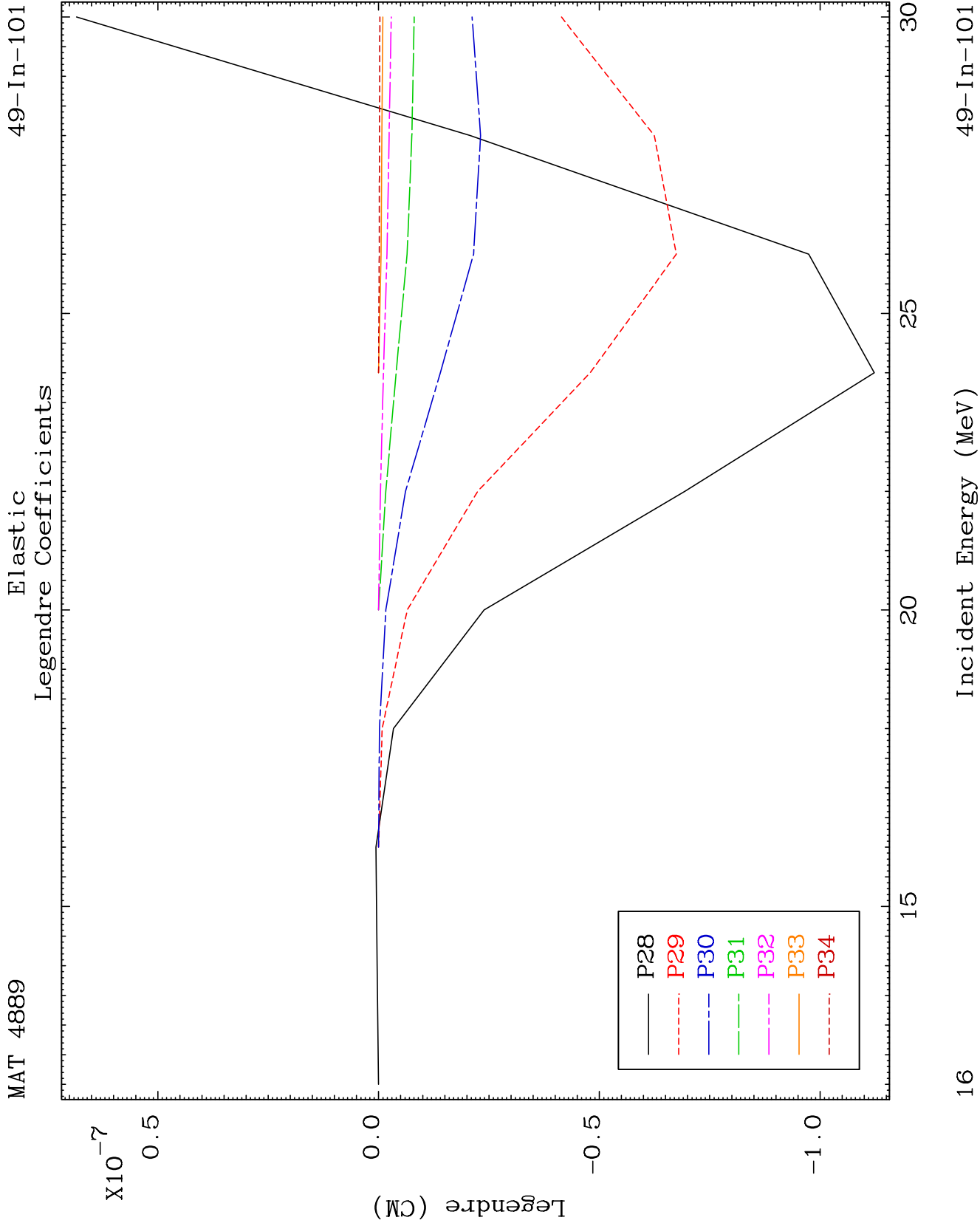








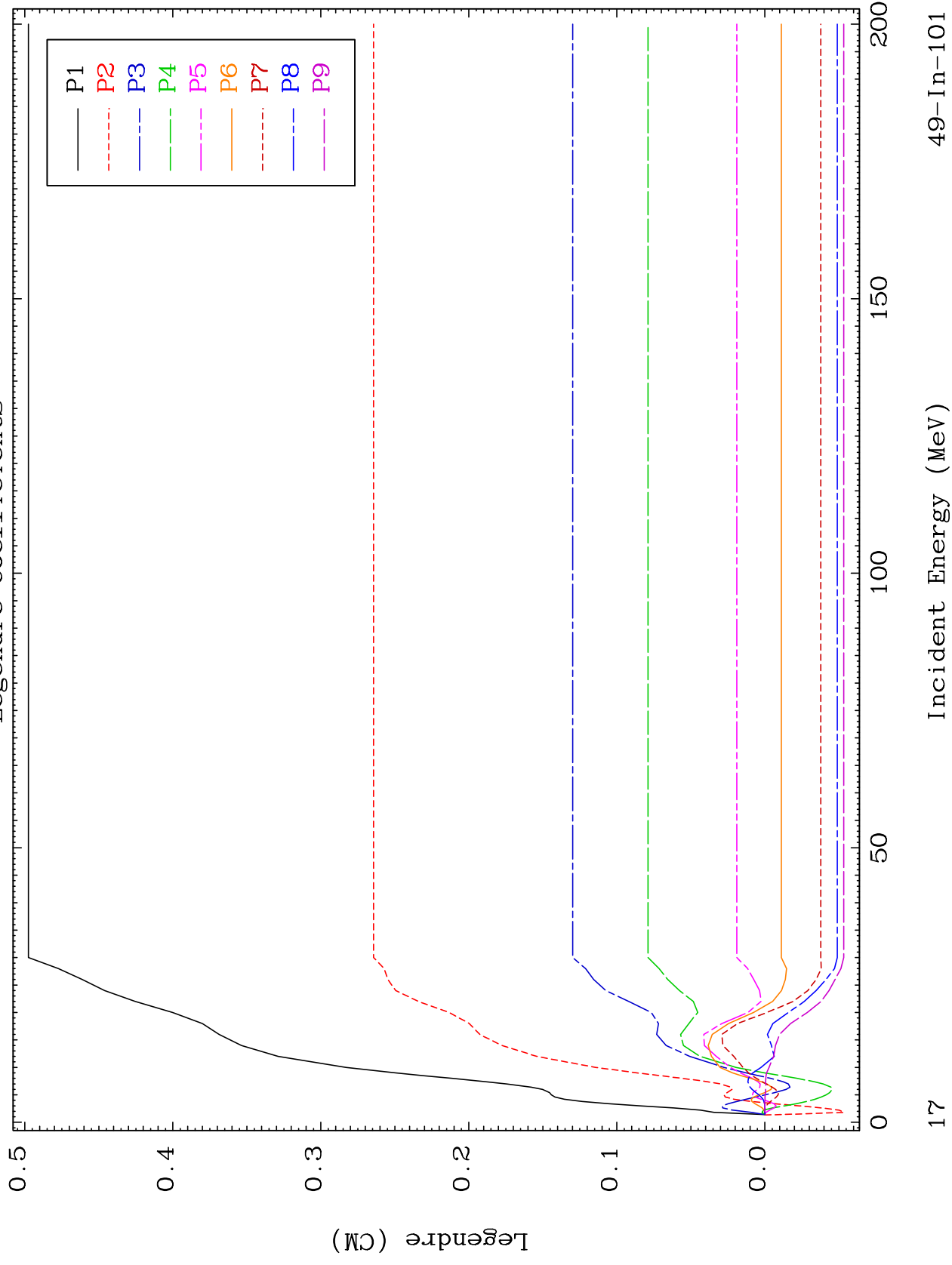


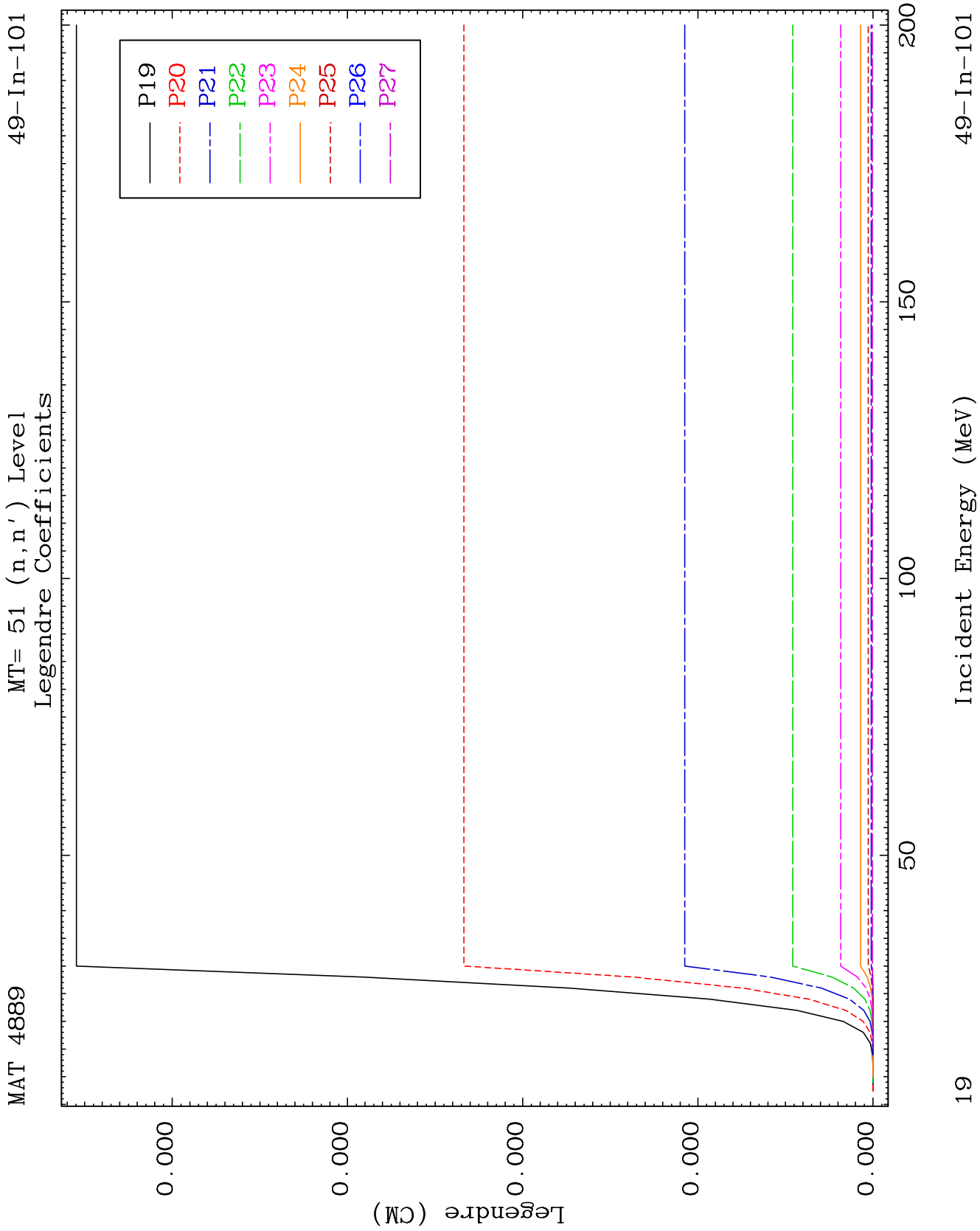


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MT= 51 (n, n') Level
Legendre Coefficients

49-In-101

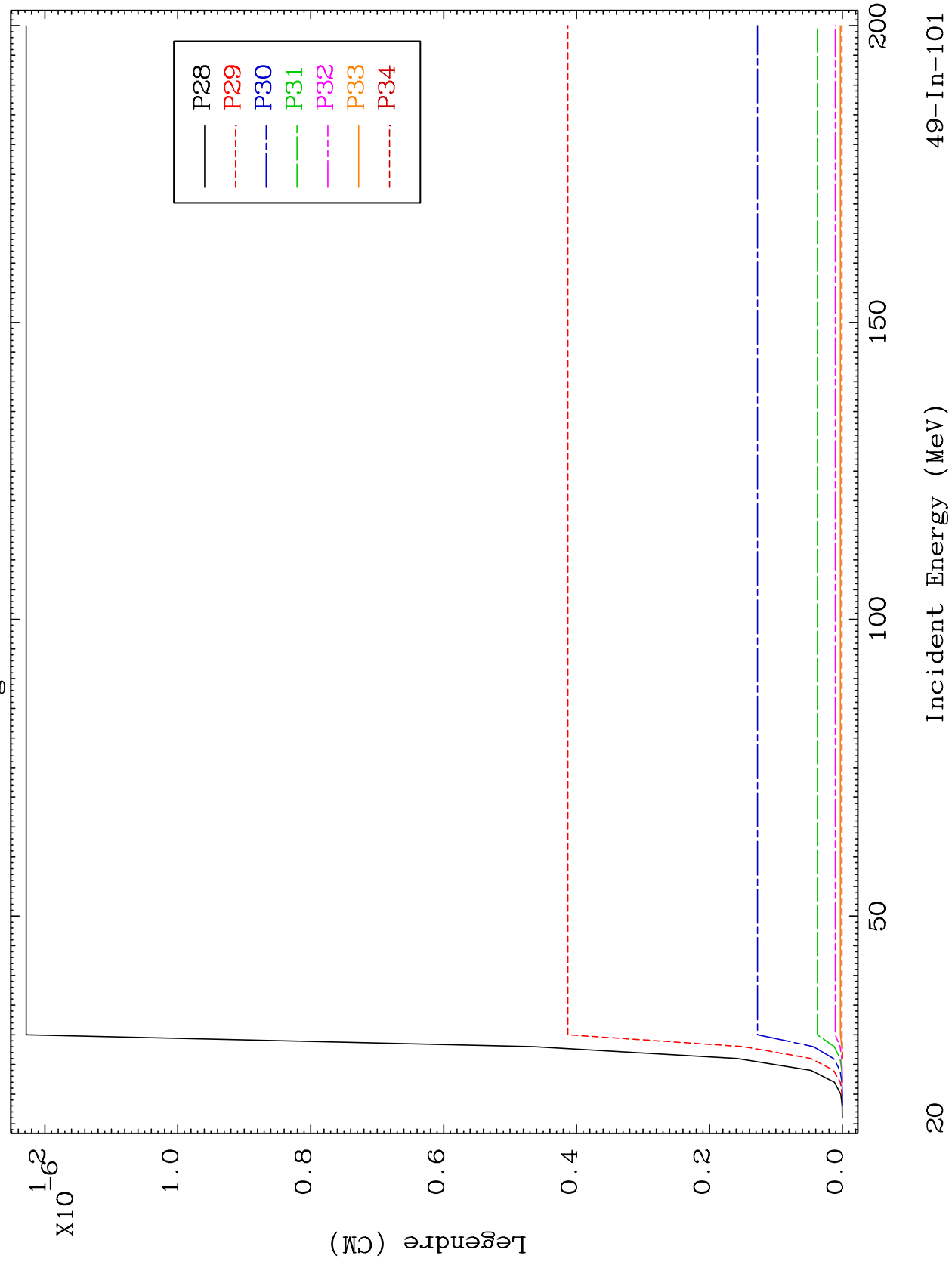


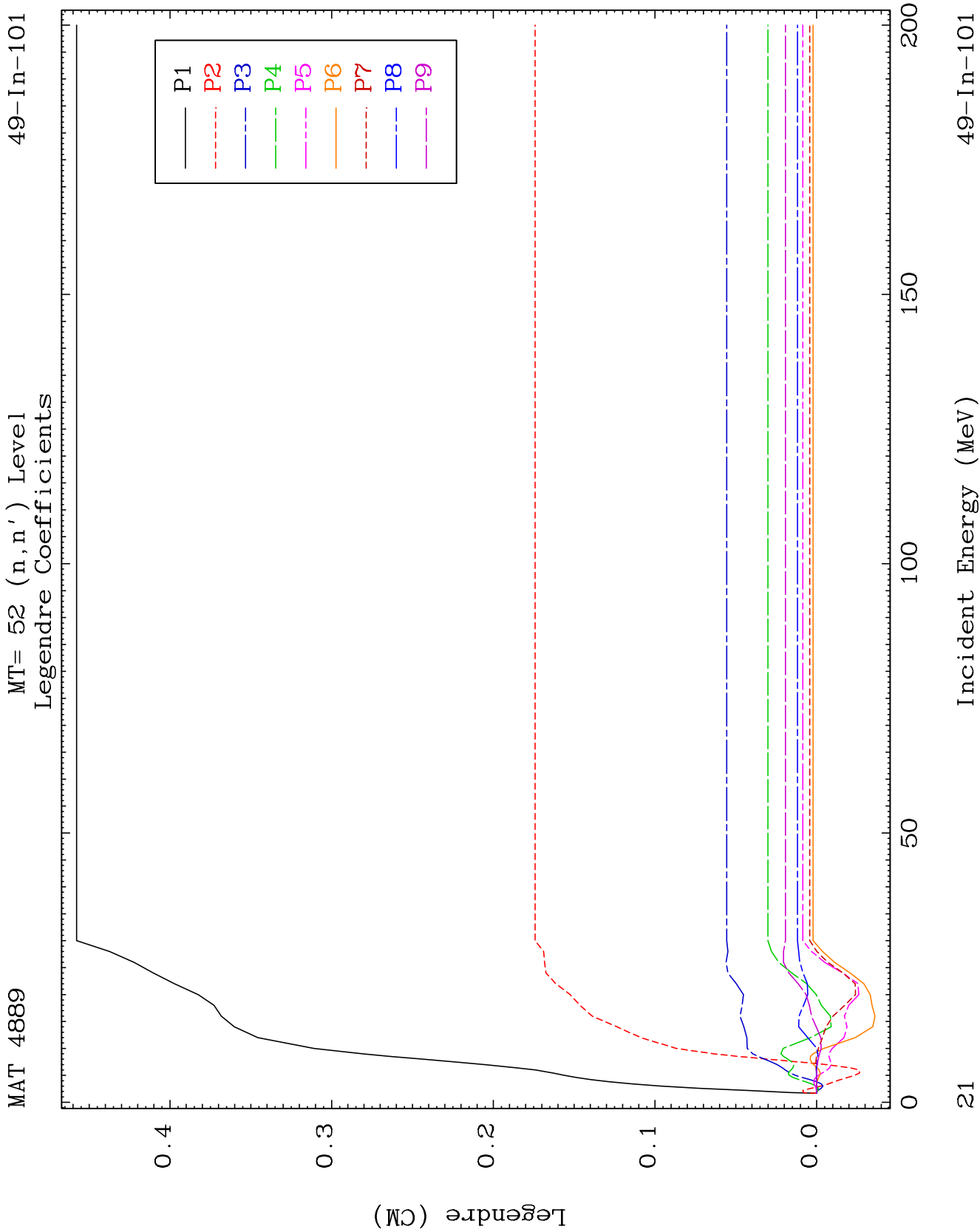


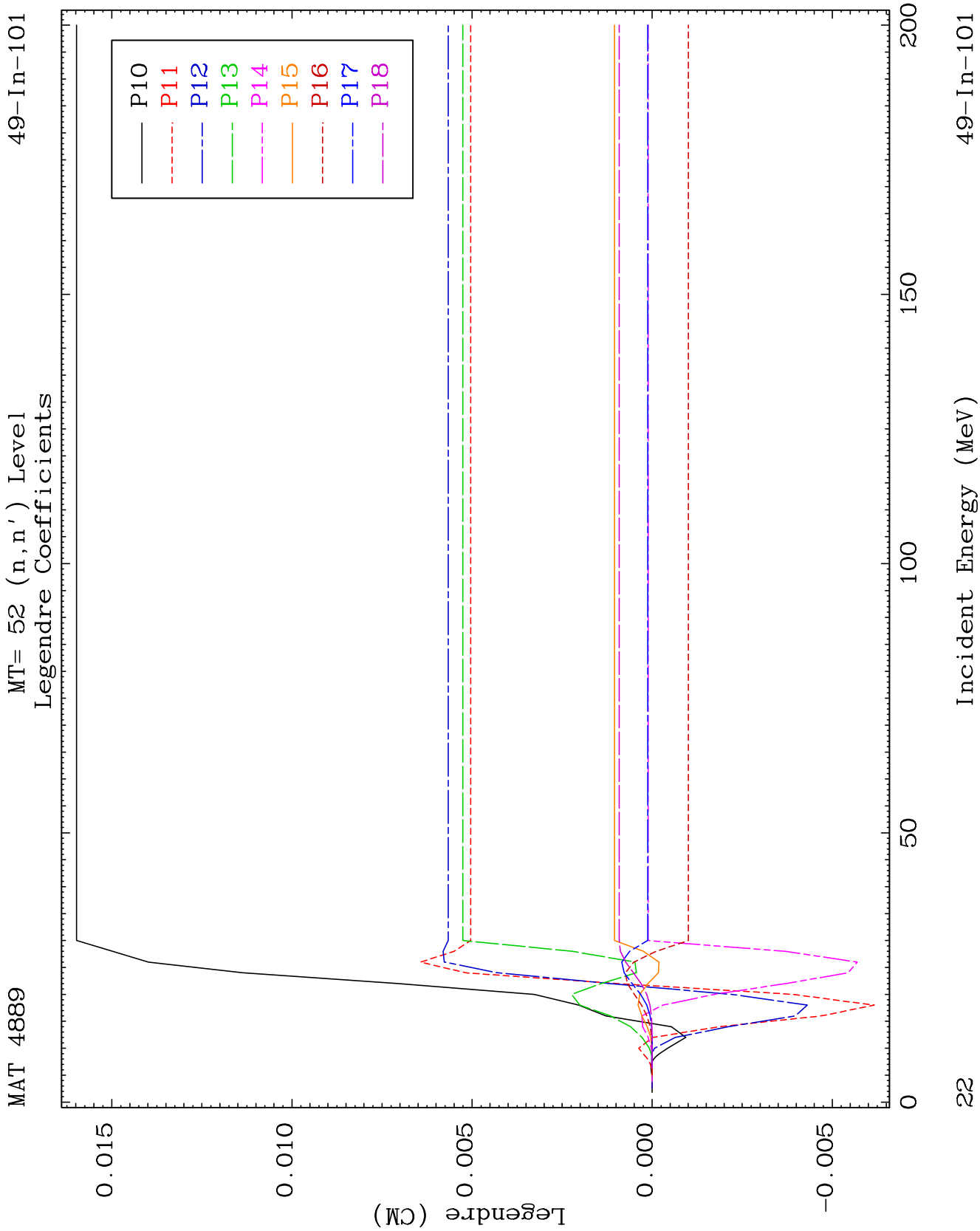
MAT 4889

MT= 51 (n, n') Level
Legendre Coefficients

49-In-101



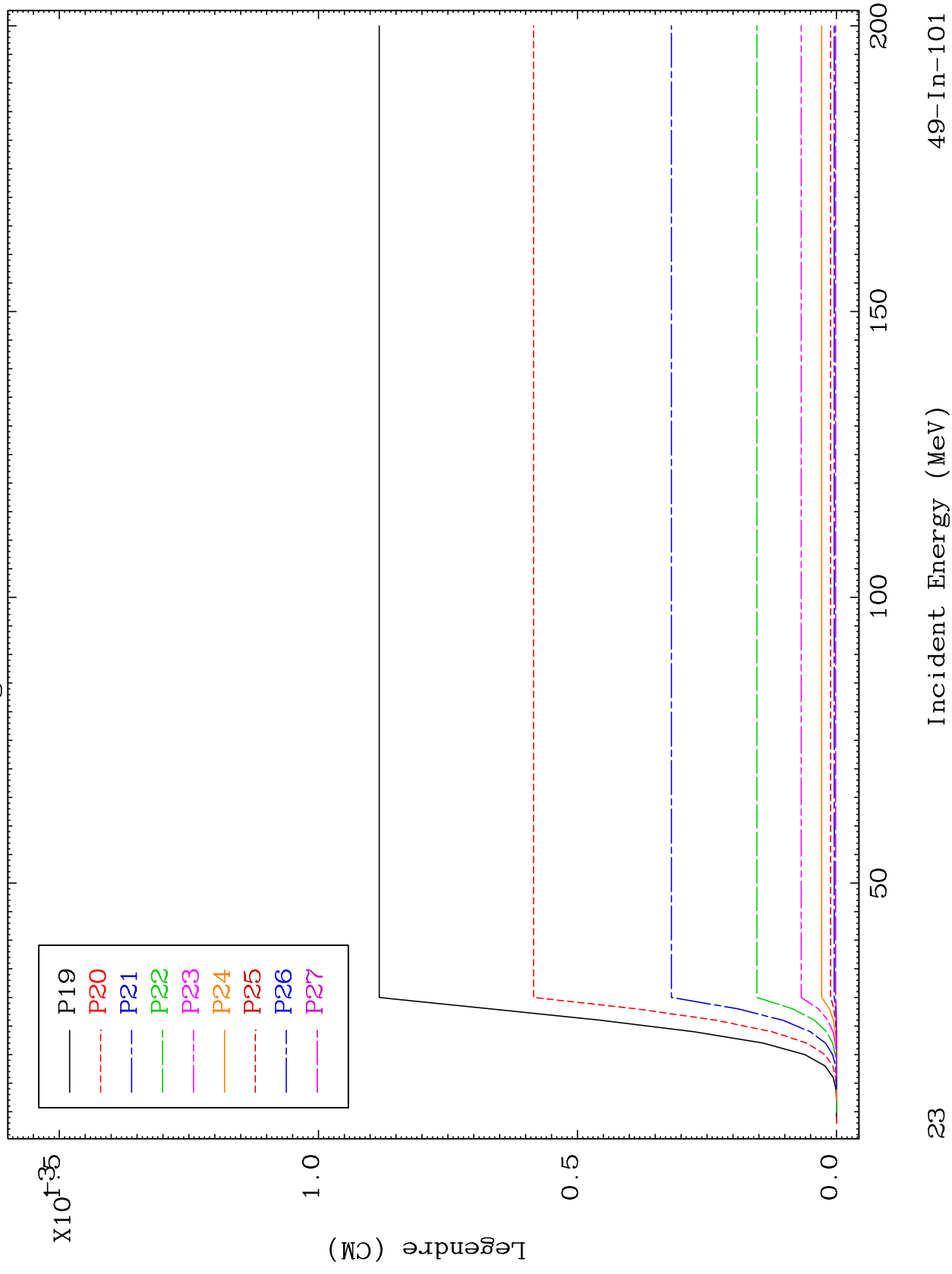


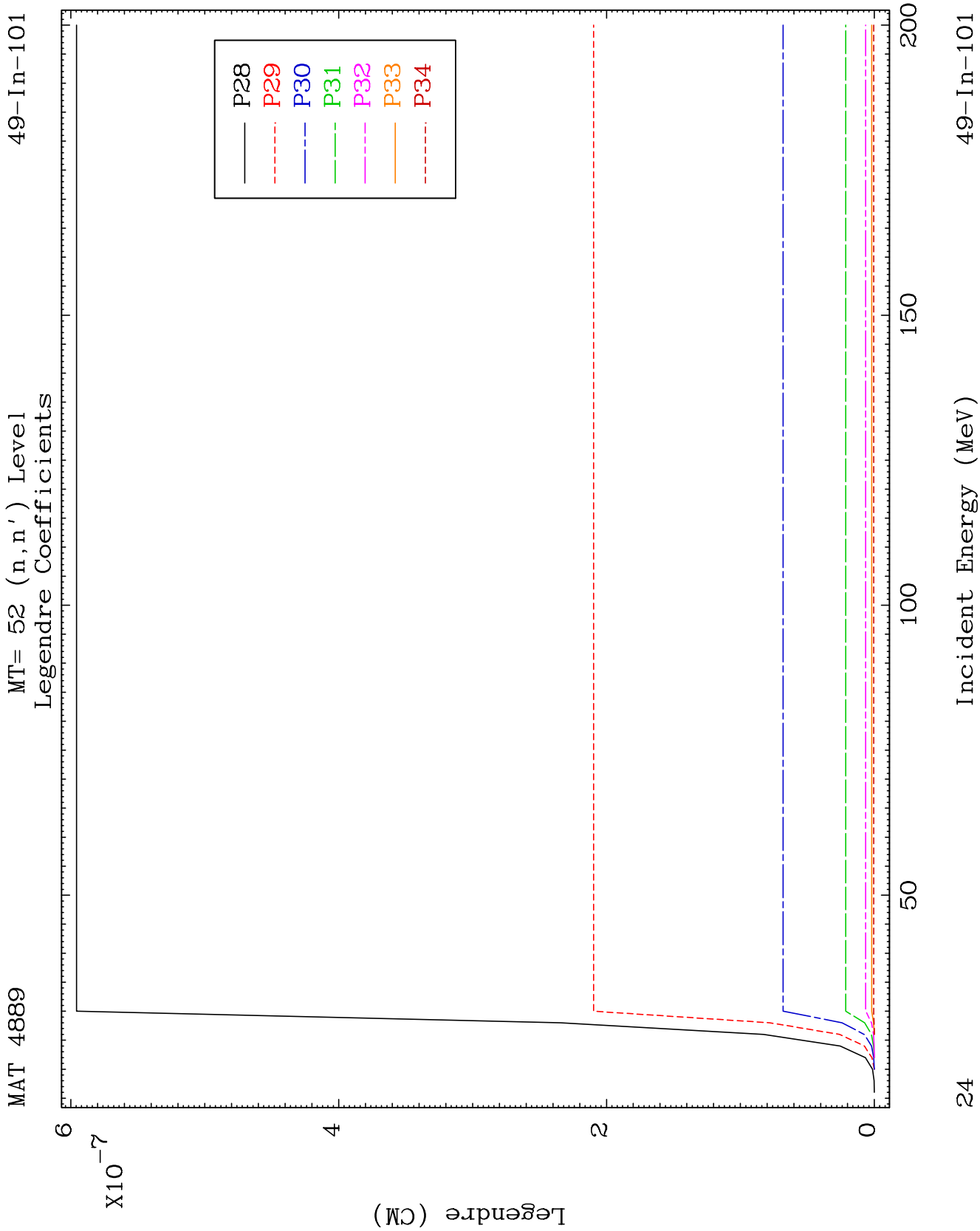


MAT 4889

MT= 52 (n, n') Level
Legendre Coefficients

49-In-101

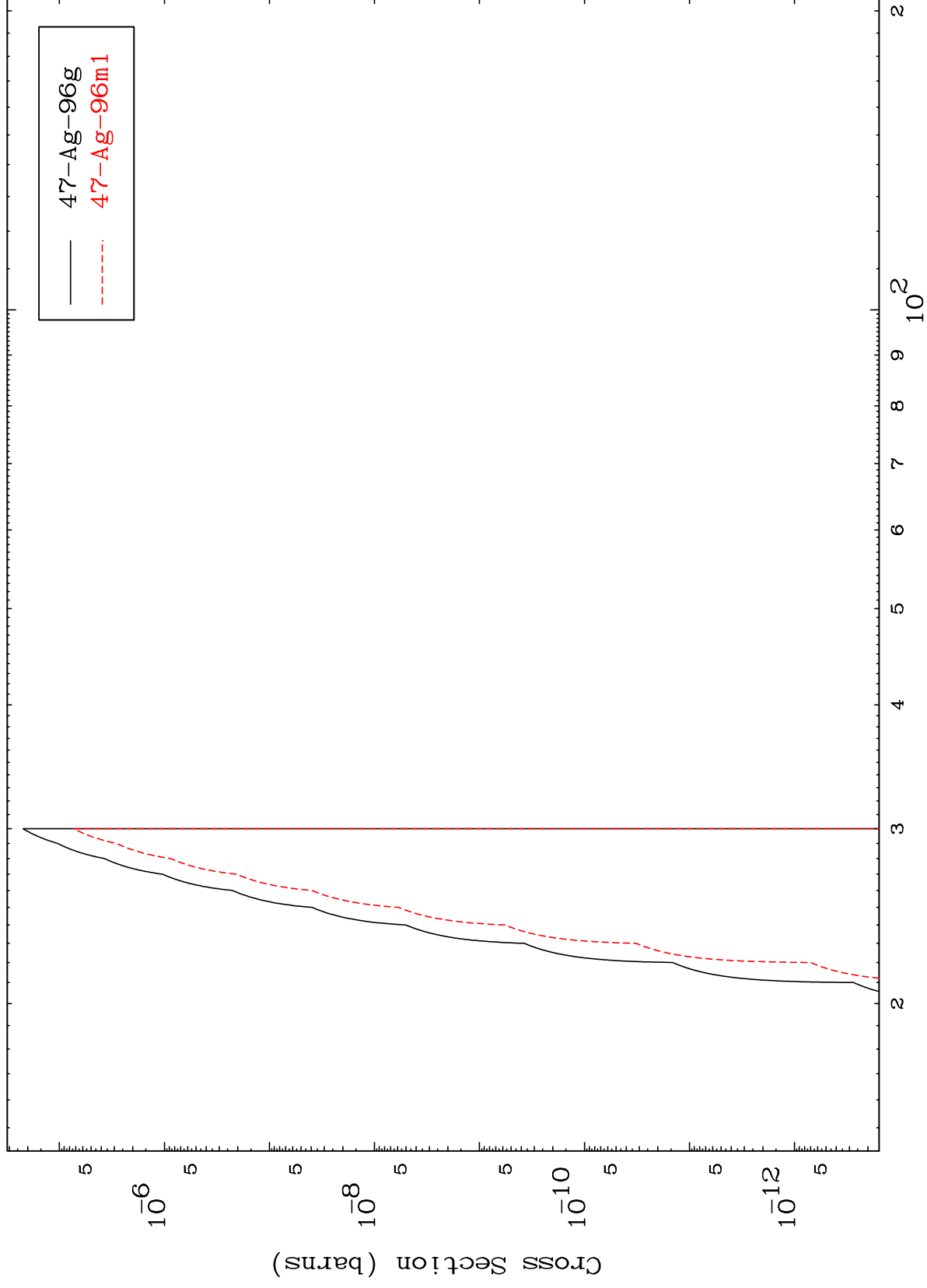




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$(n,2n) \alpha$
Radionuclide Production Cross Section



25

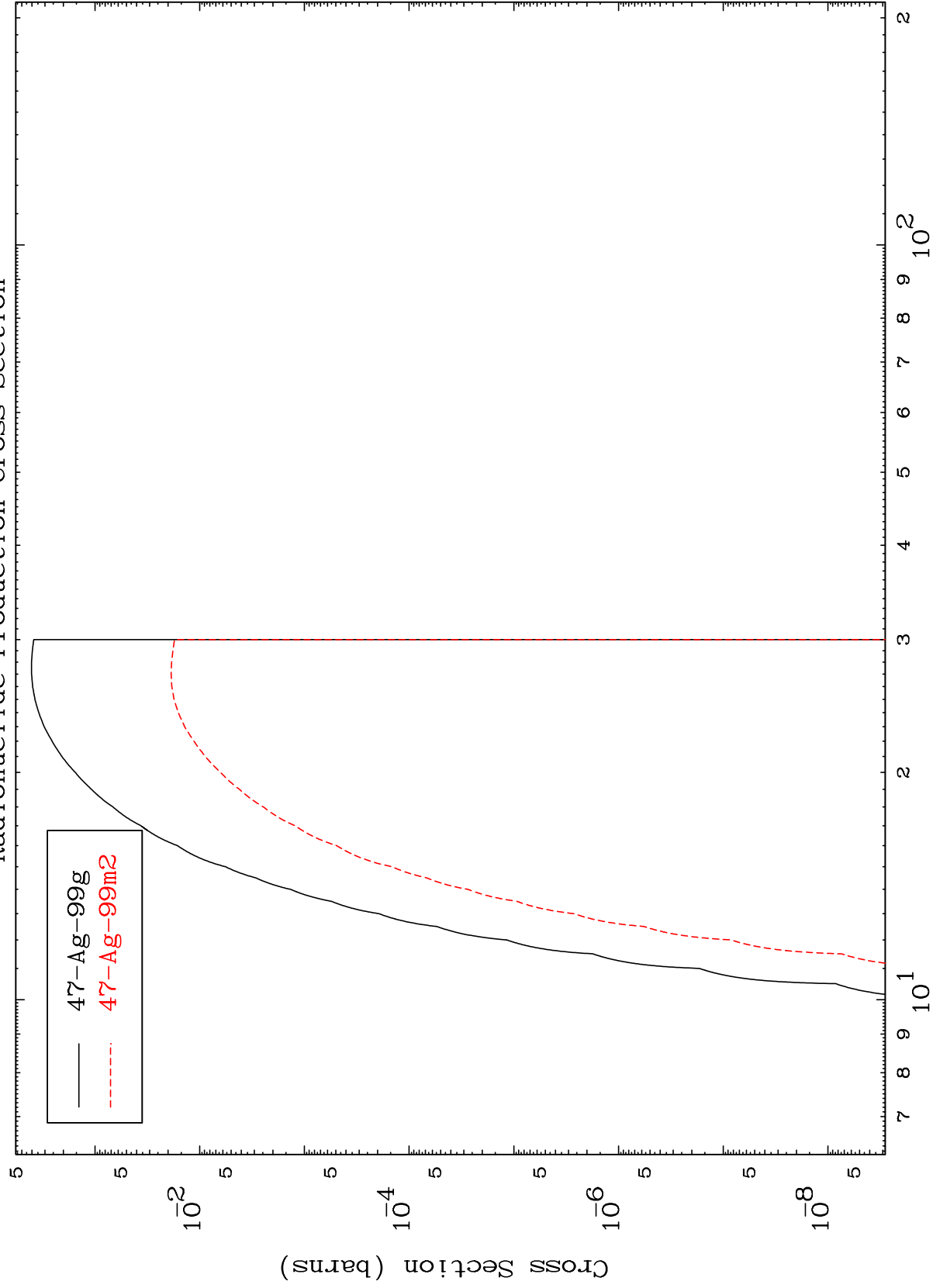
Incident Energy (MeV)

49-In-101

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49-In-101

(n,2n) p
Radionuclide Production Cross Section



26

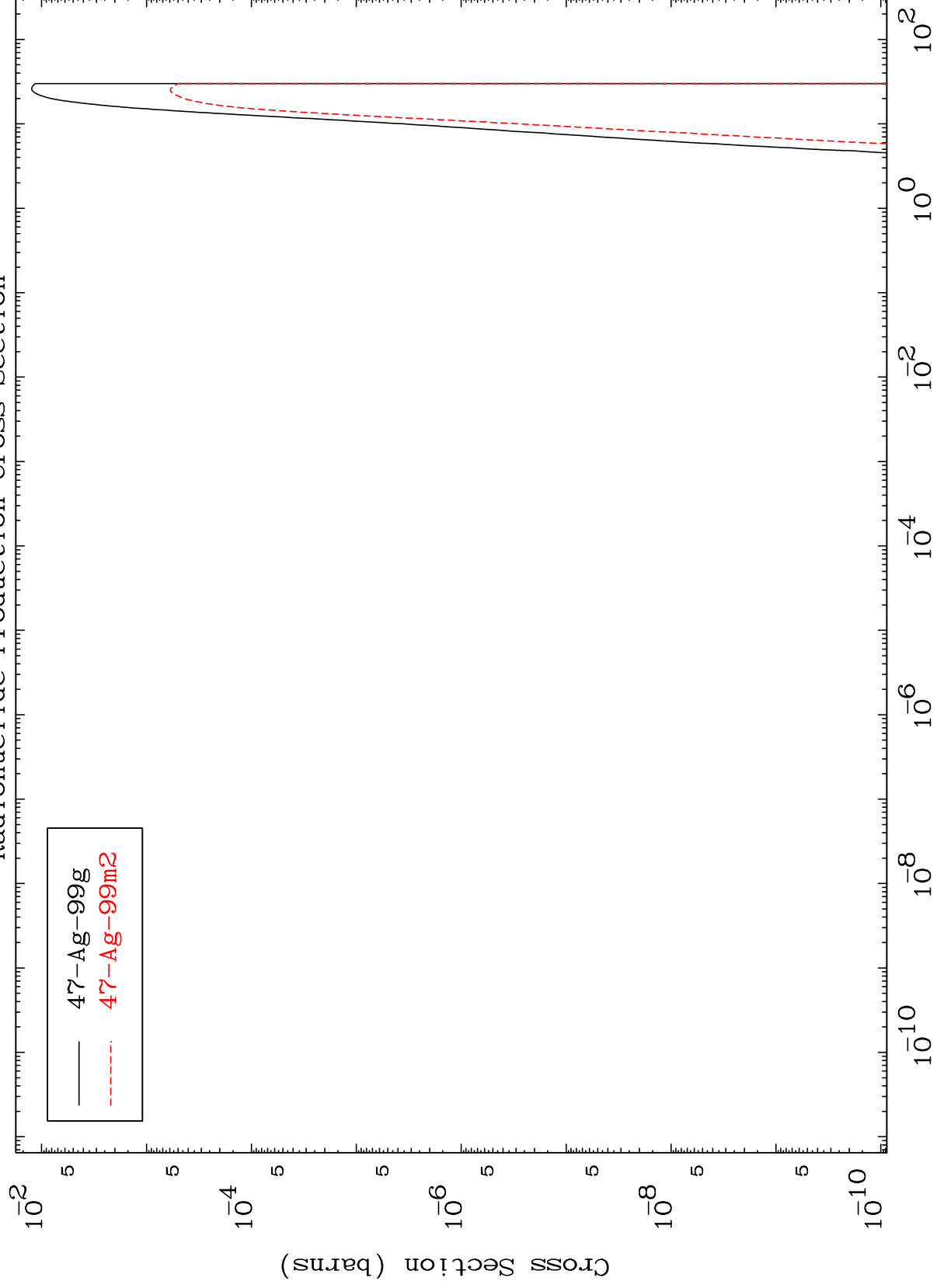
Incident Energy (MeV)

49-In-101

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Radionuclide Production Cross Section
(n,He-3)



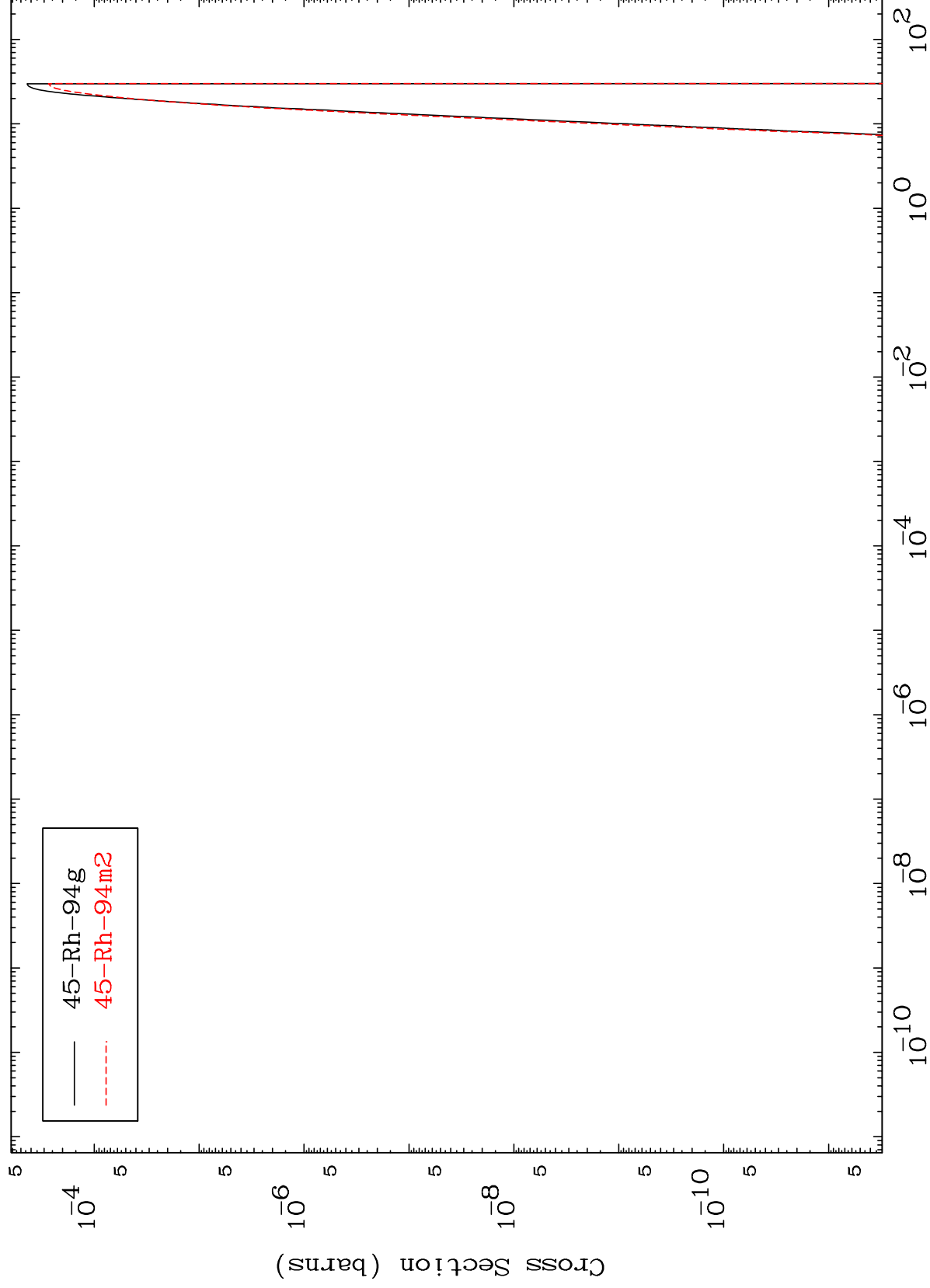
27

49-In-101

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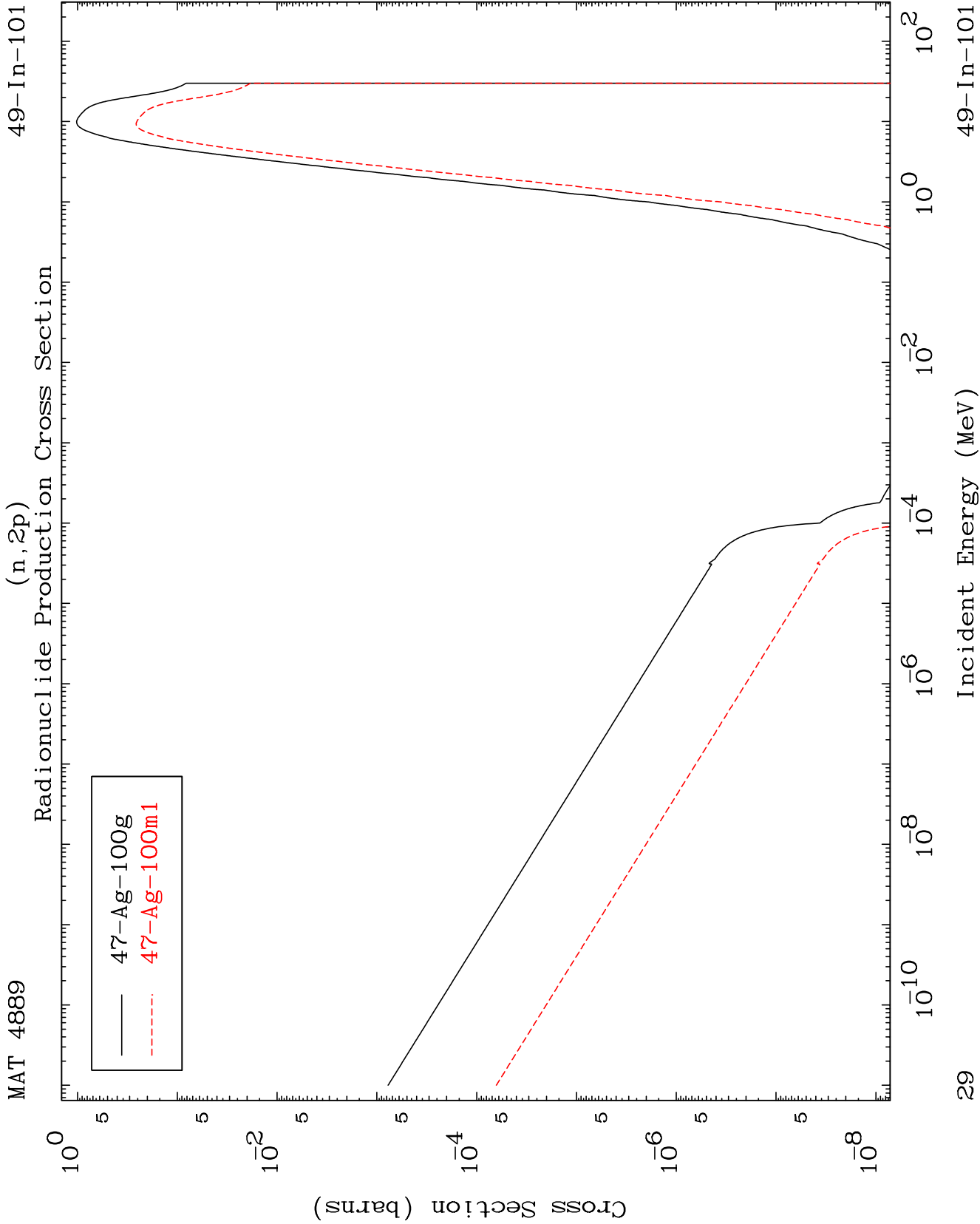
49-In-101

Radionuclide Production Cross Section
(n,2 α)



28

49-In-101

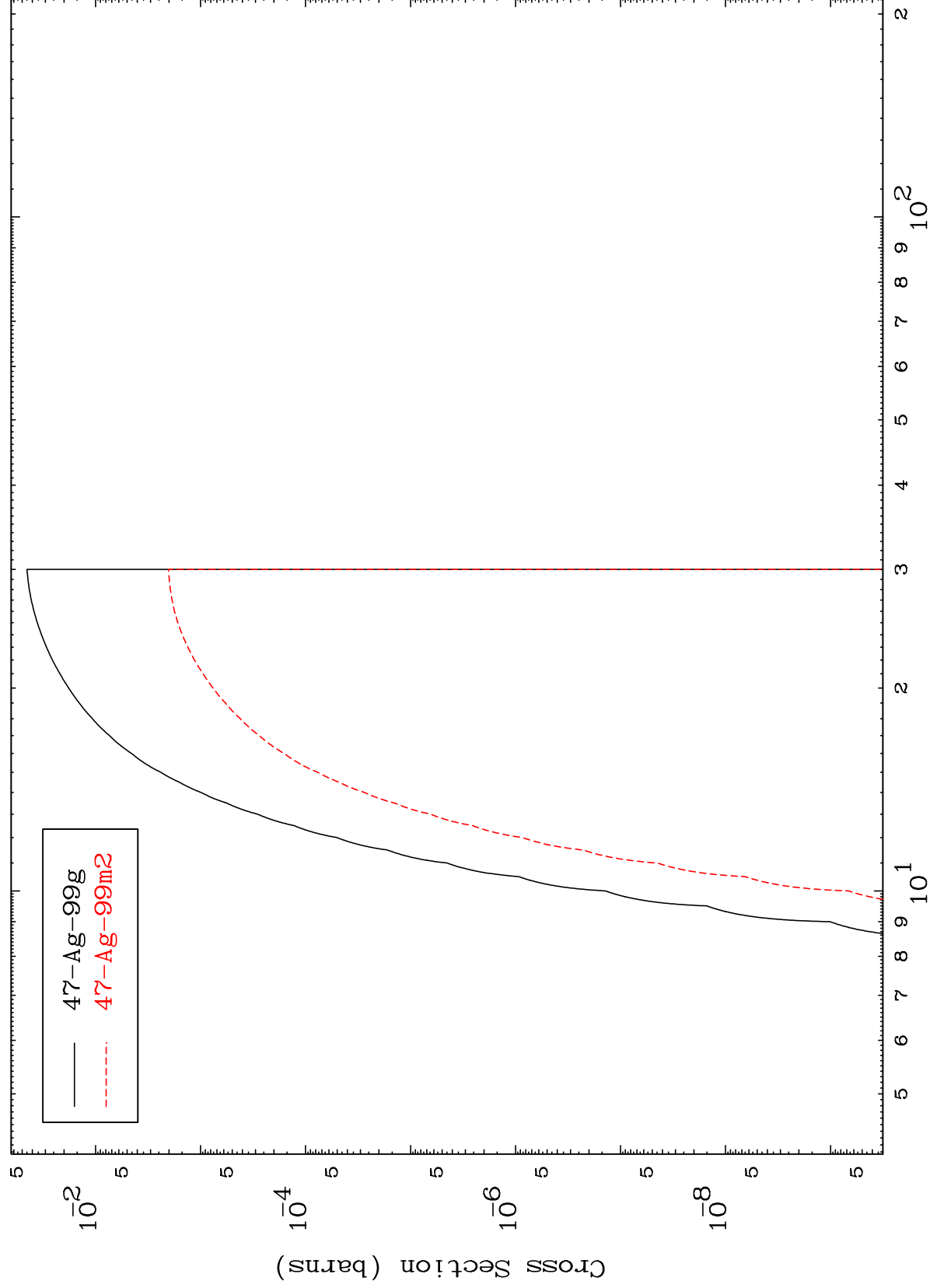


MAT 4889

(n,p) d

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Radionuclide Production Cross Section



30

Incident Energy (MeV)

49-In-101