

Program EVALPLOT
(Version 2017-1)

by

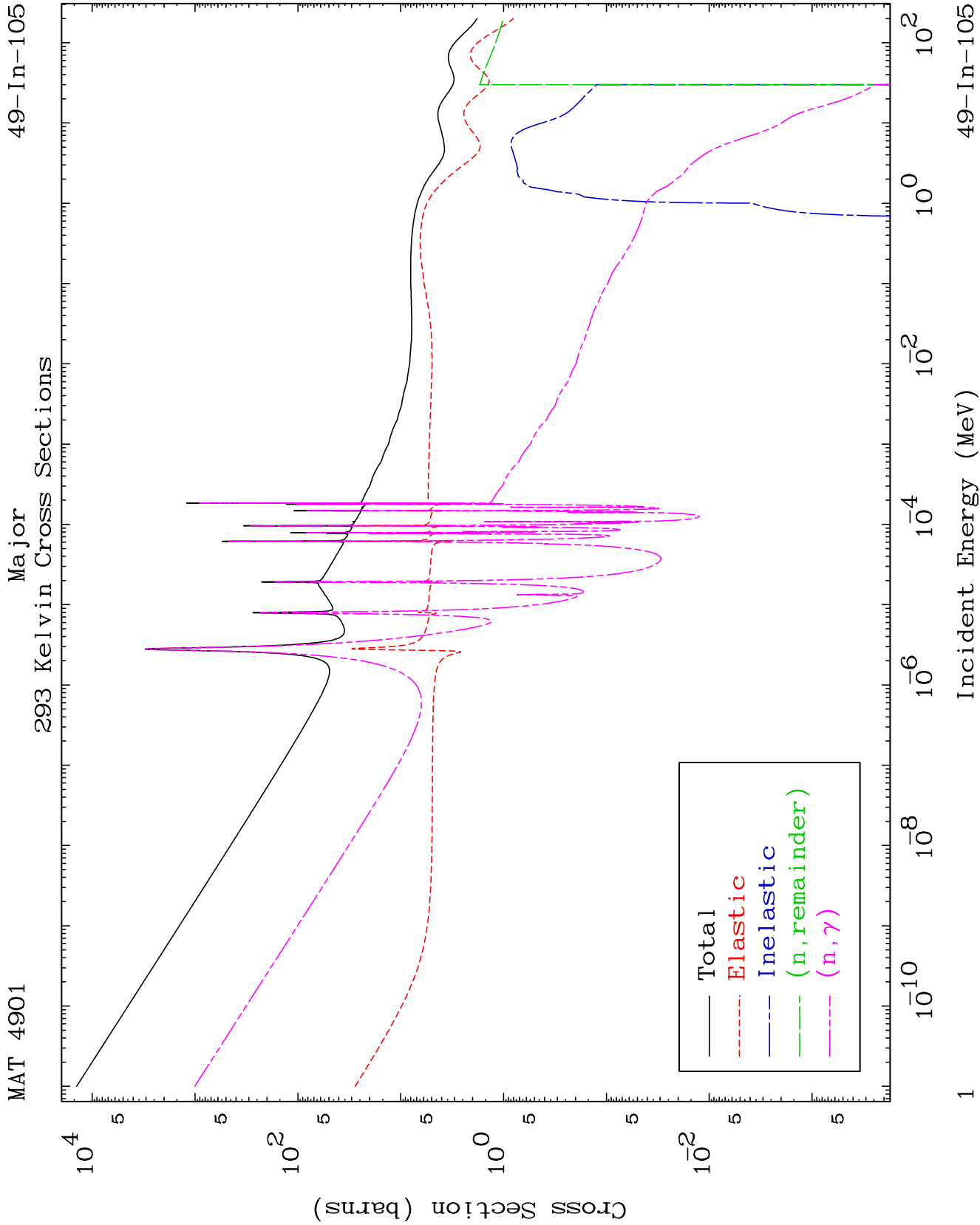
Dermott E. Cullen
(Present Contact Information)

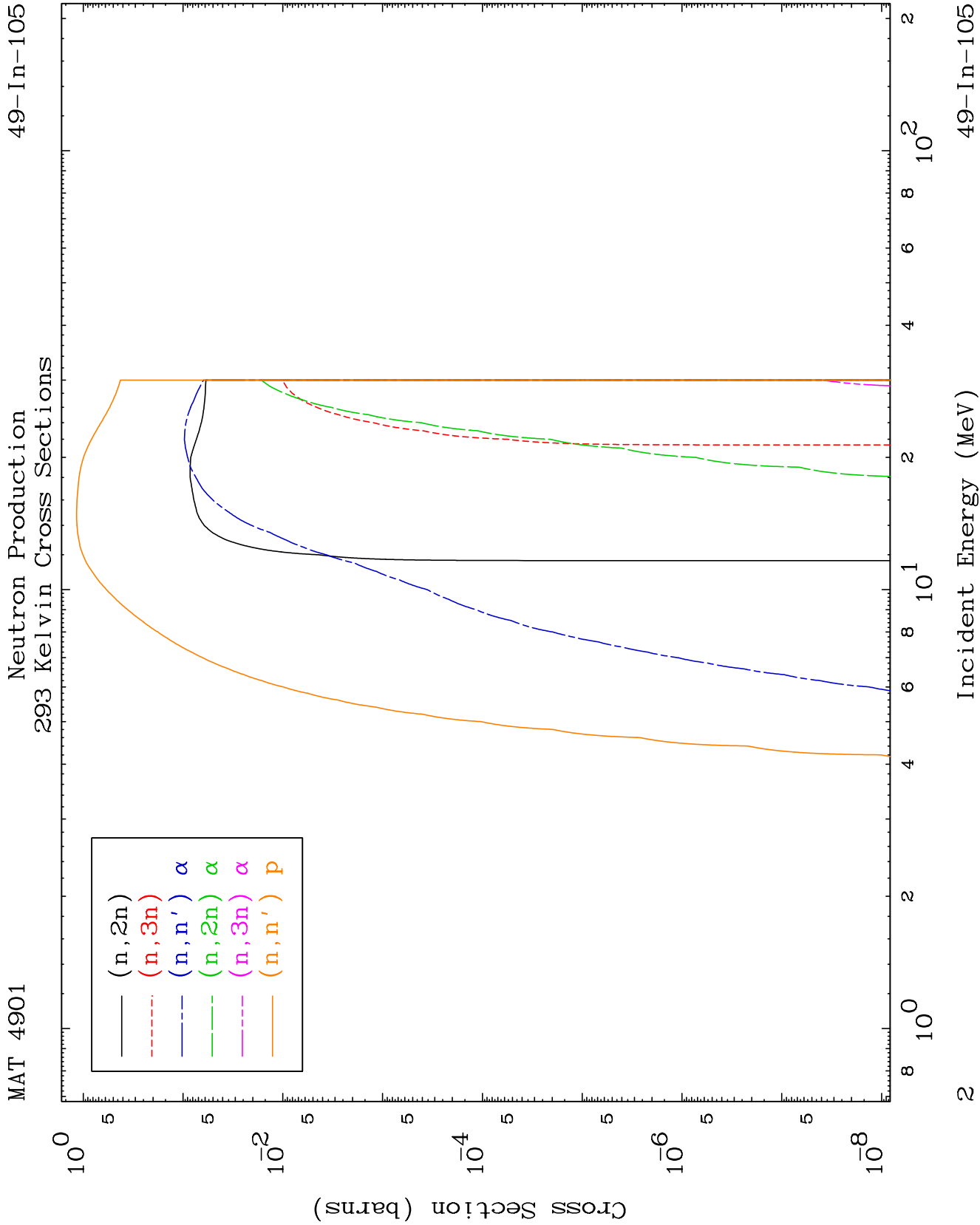
Dermott E. Cullen
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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

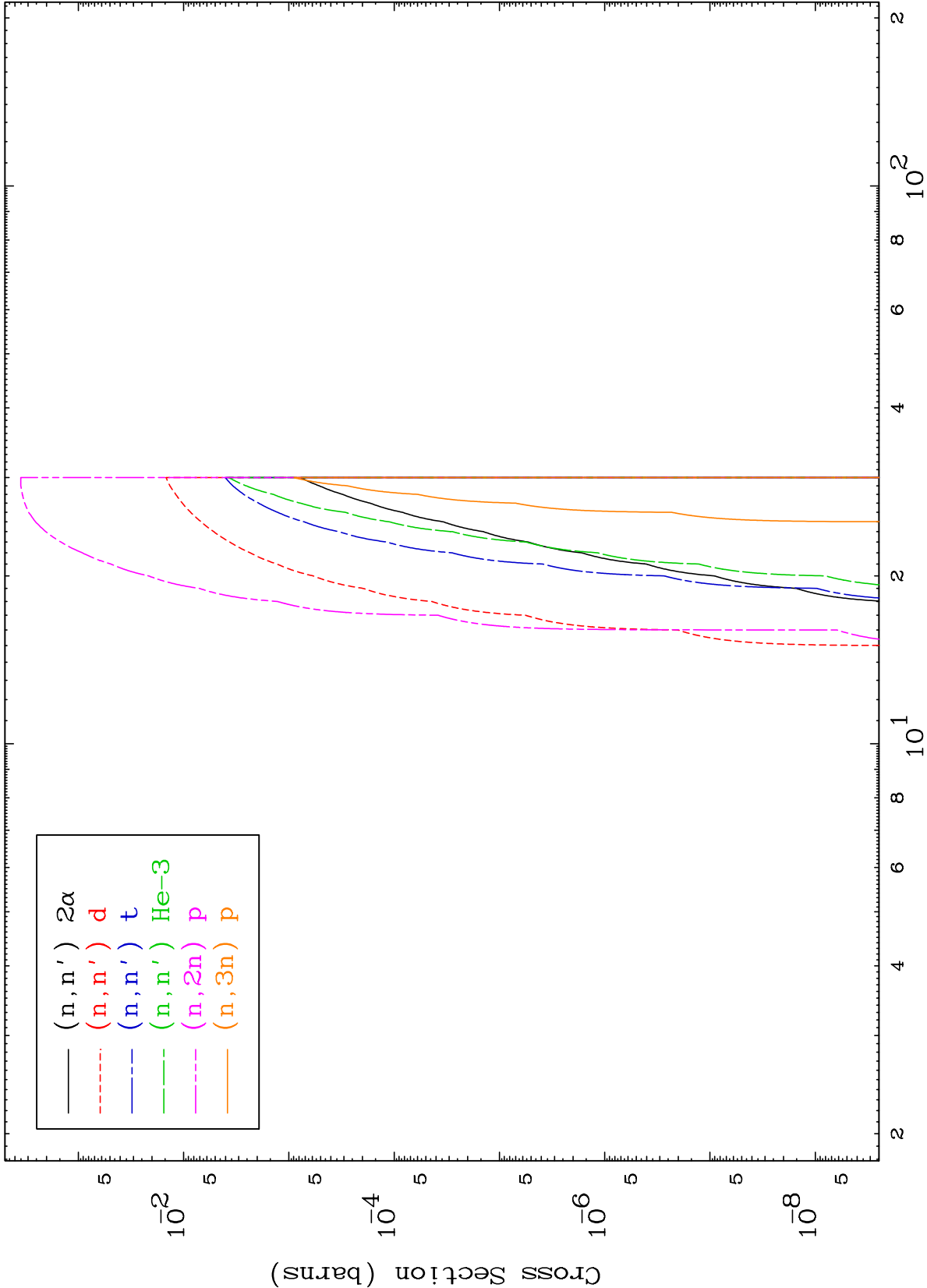


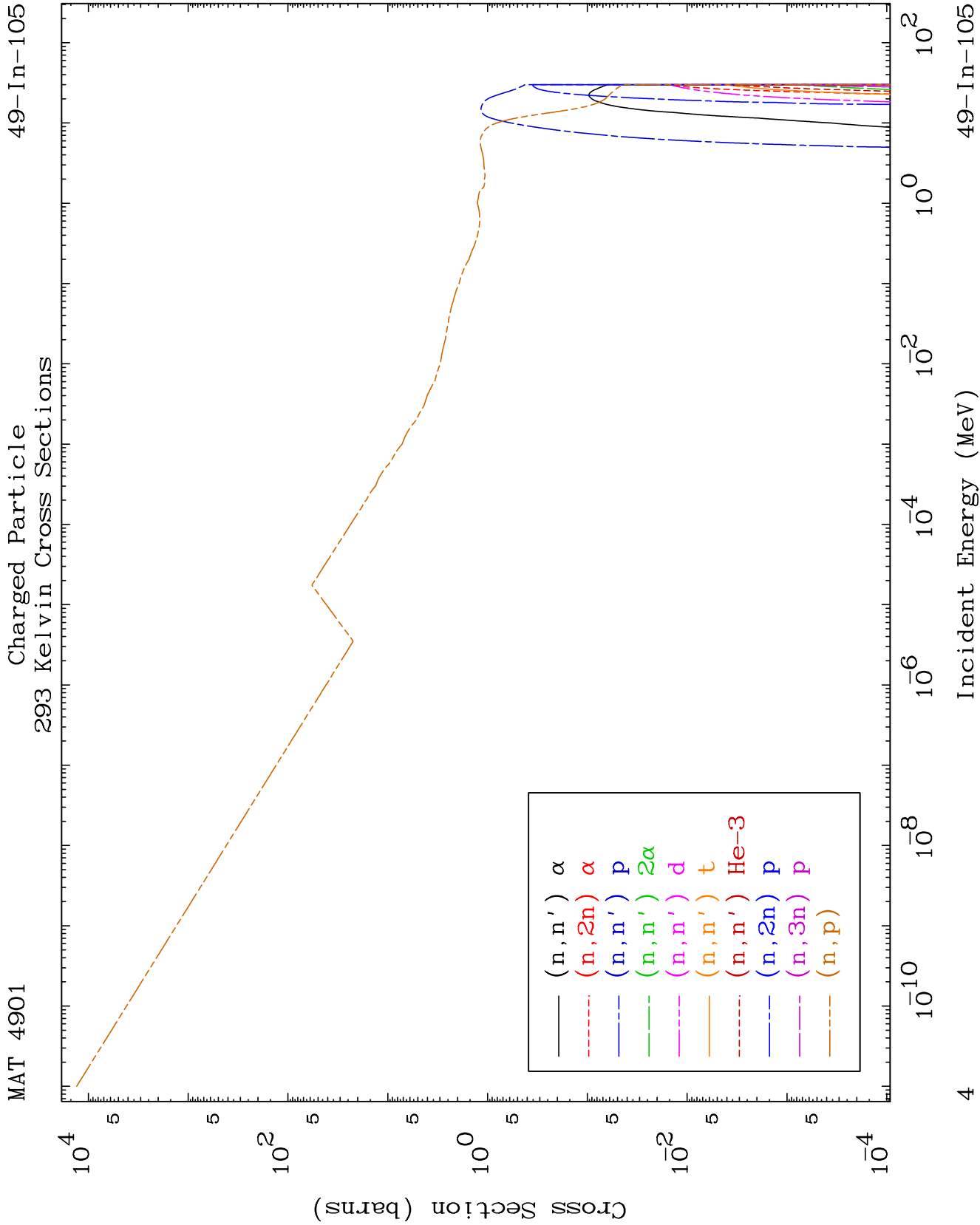


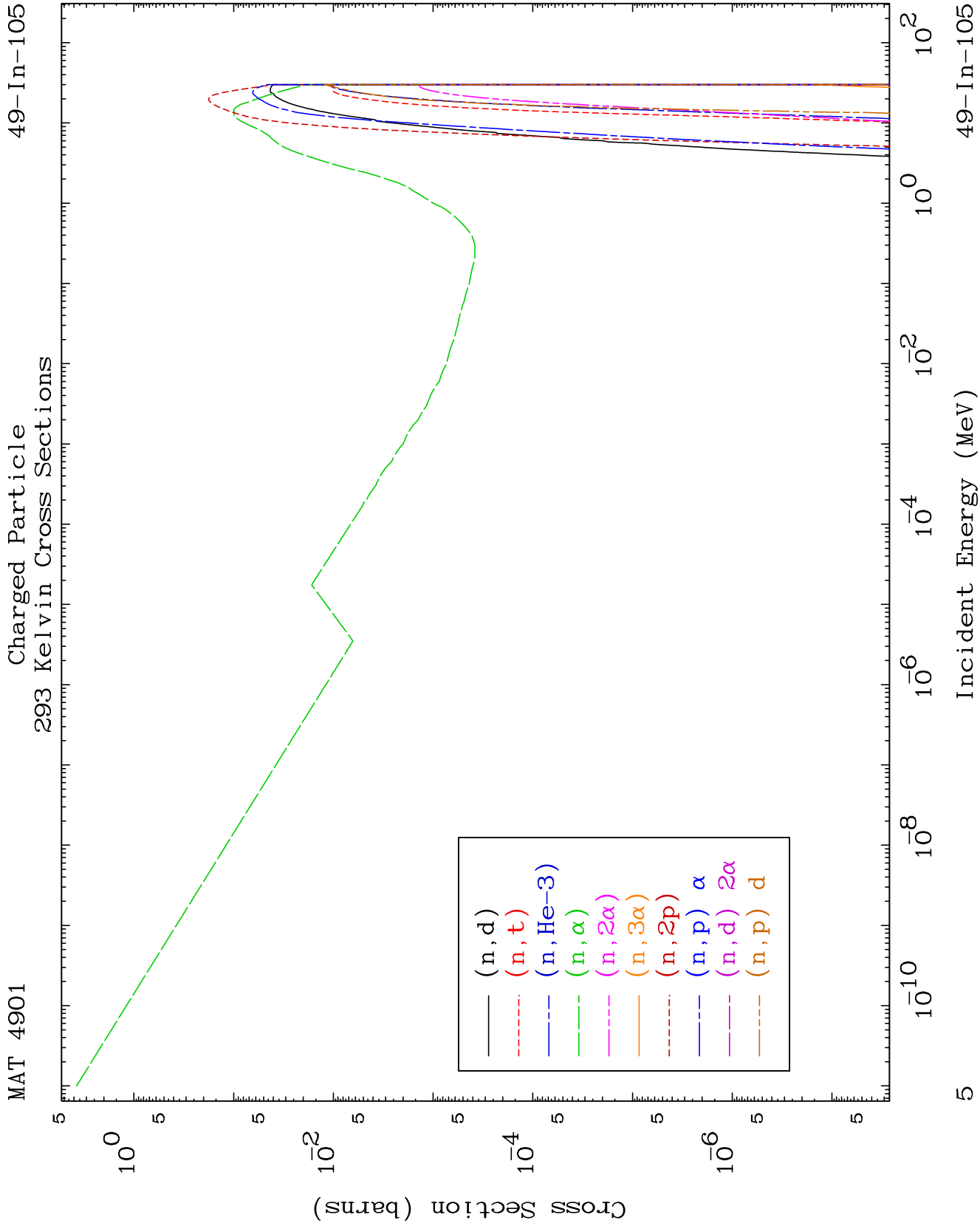
MAT 4901

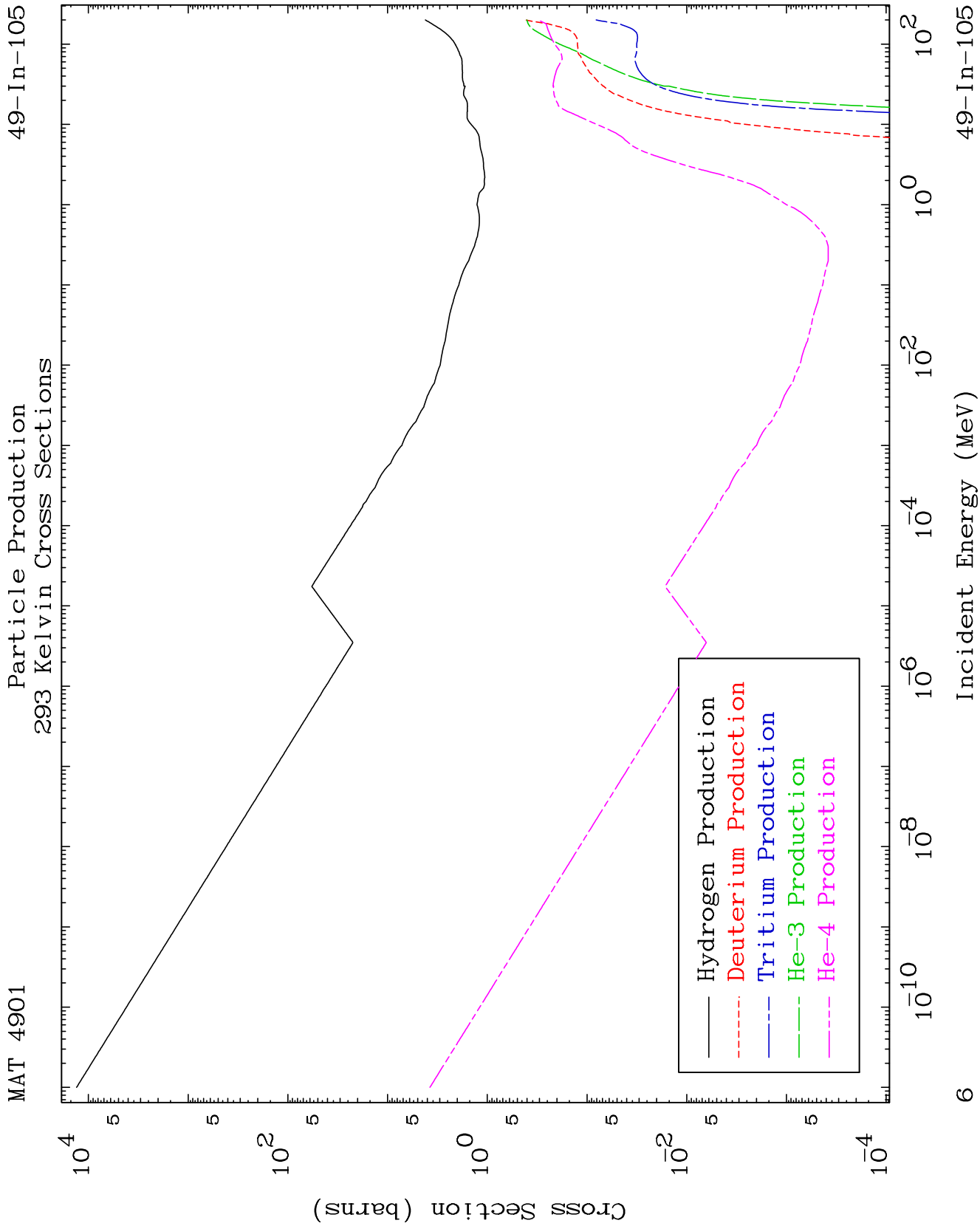
Neutron Production
293 Kelvin Cross Sections

49-In-105







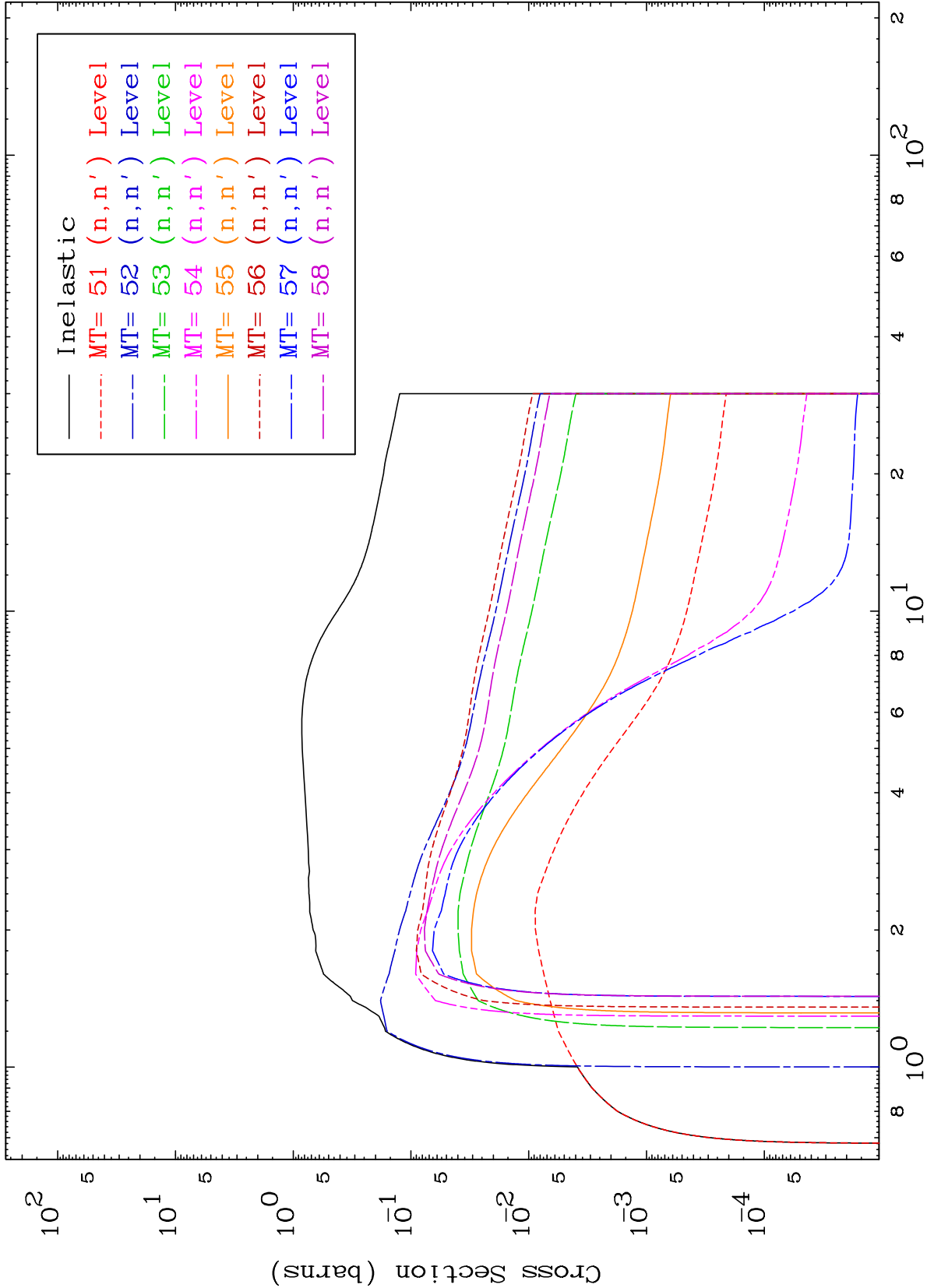


MAT 4901

(n,n') Level

293 Kelvin Cross Sections

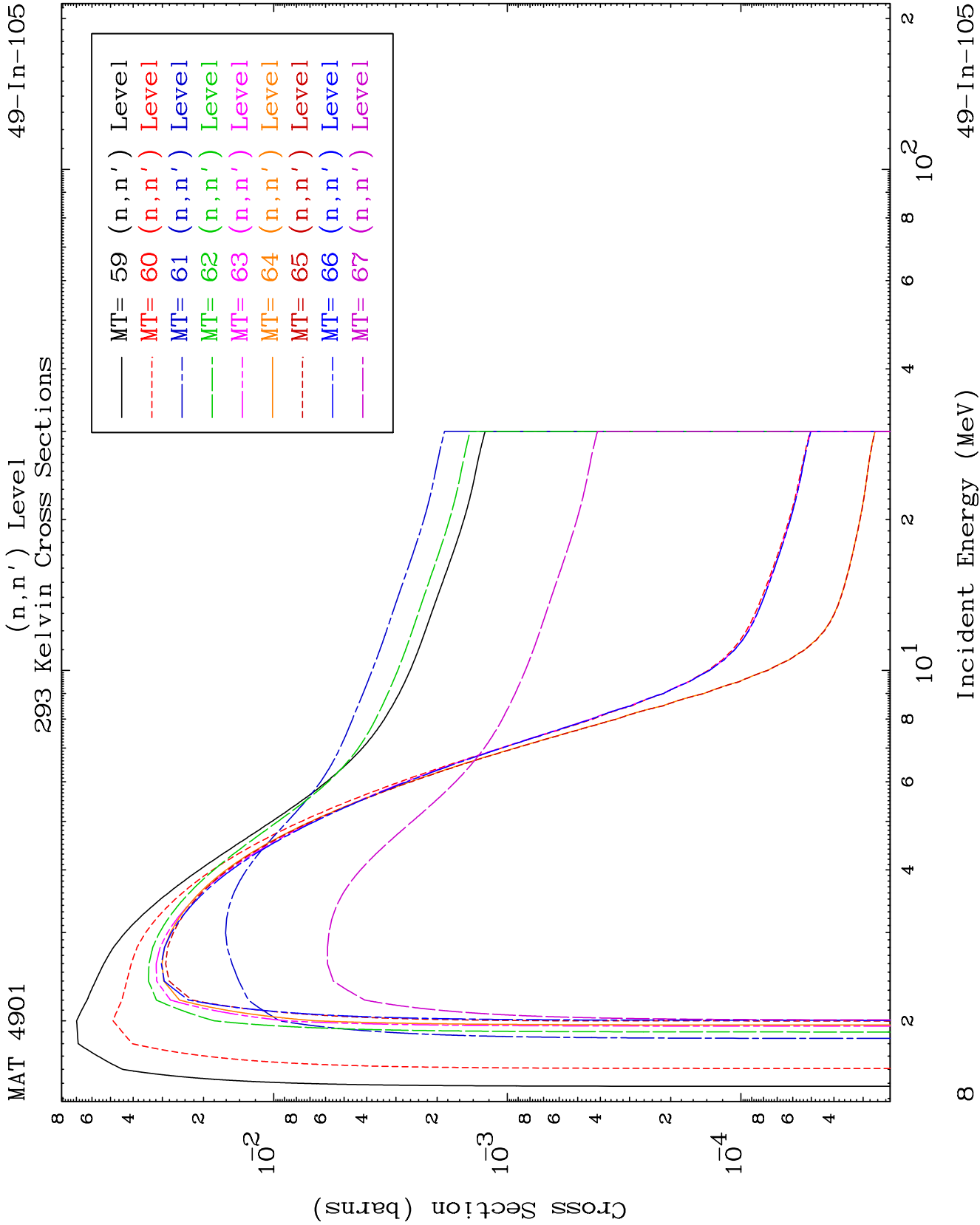
49-In-105

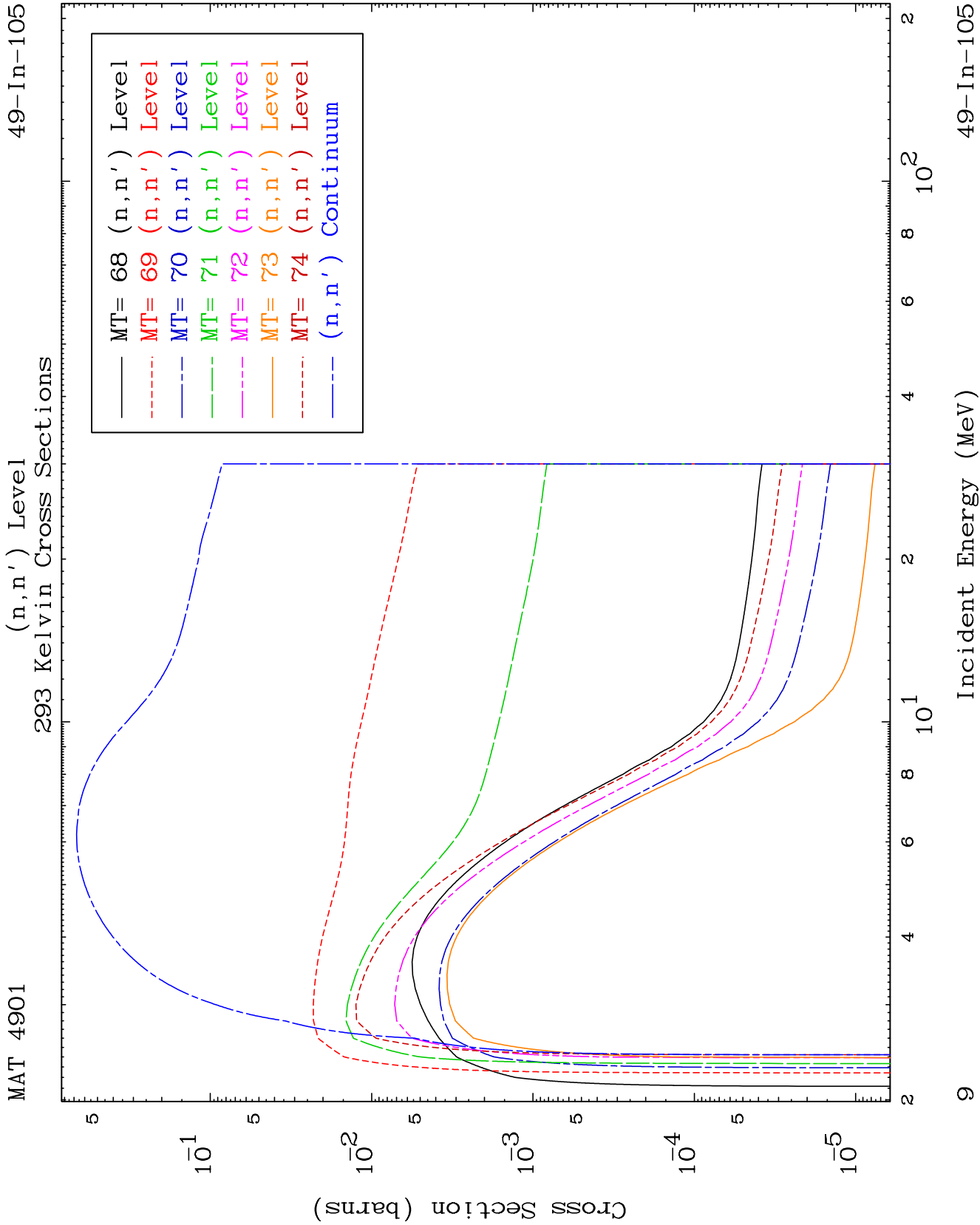


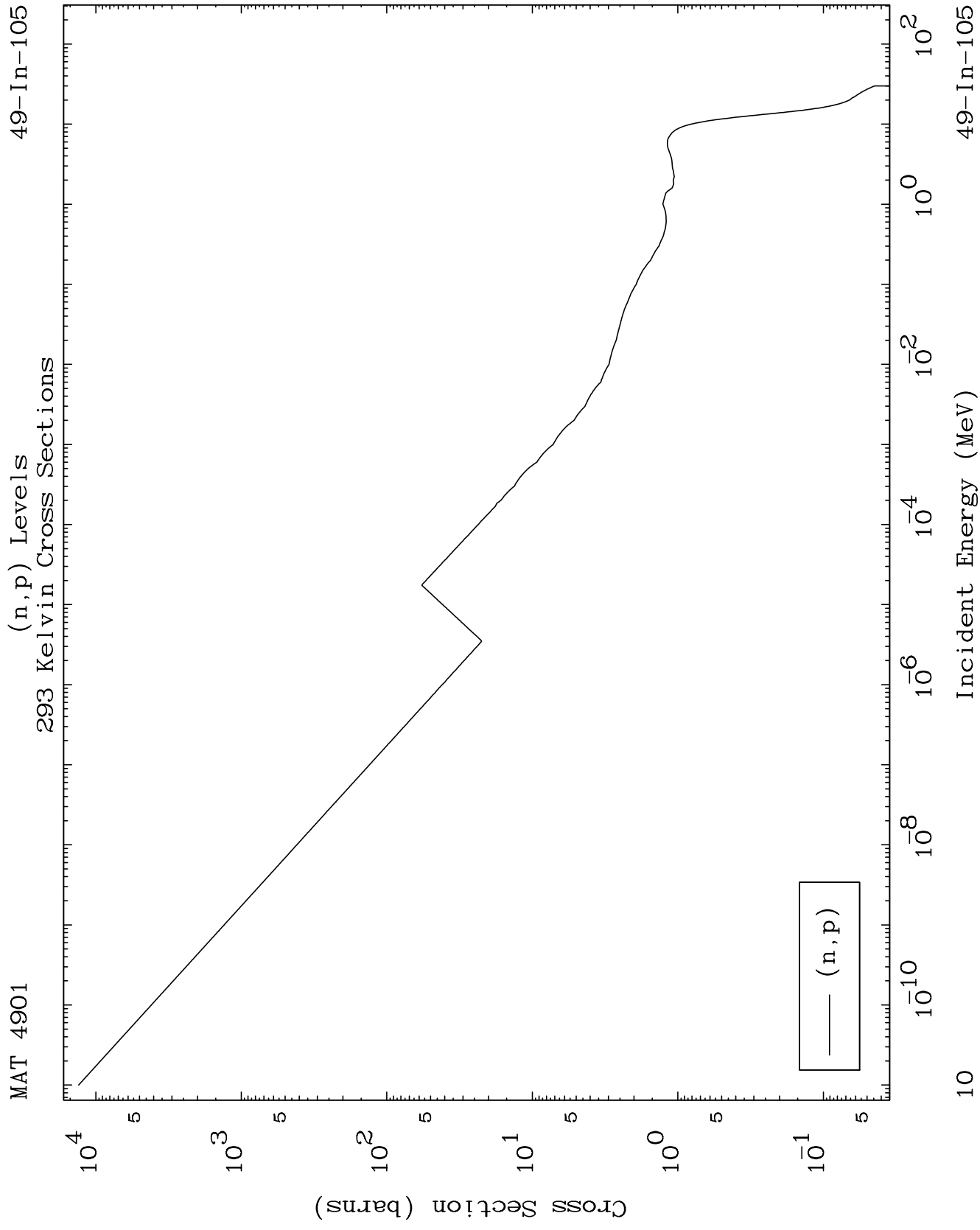
7

Incident Energy (MeV)

49-In-105



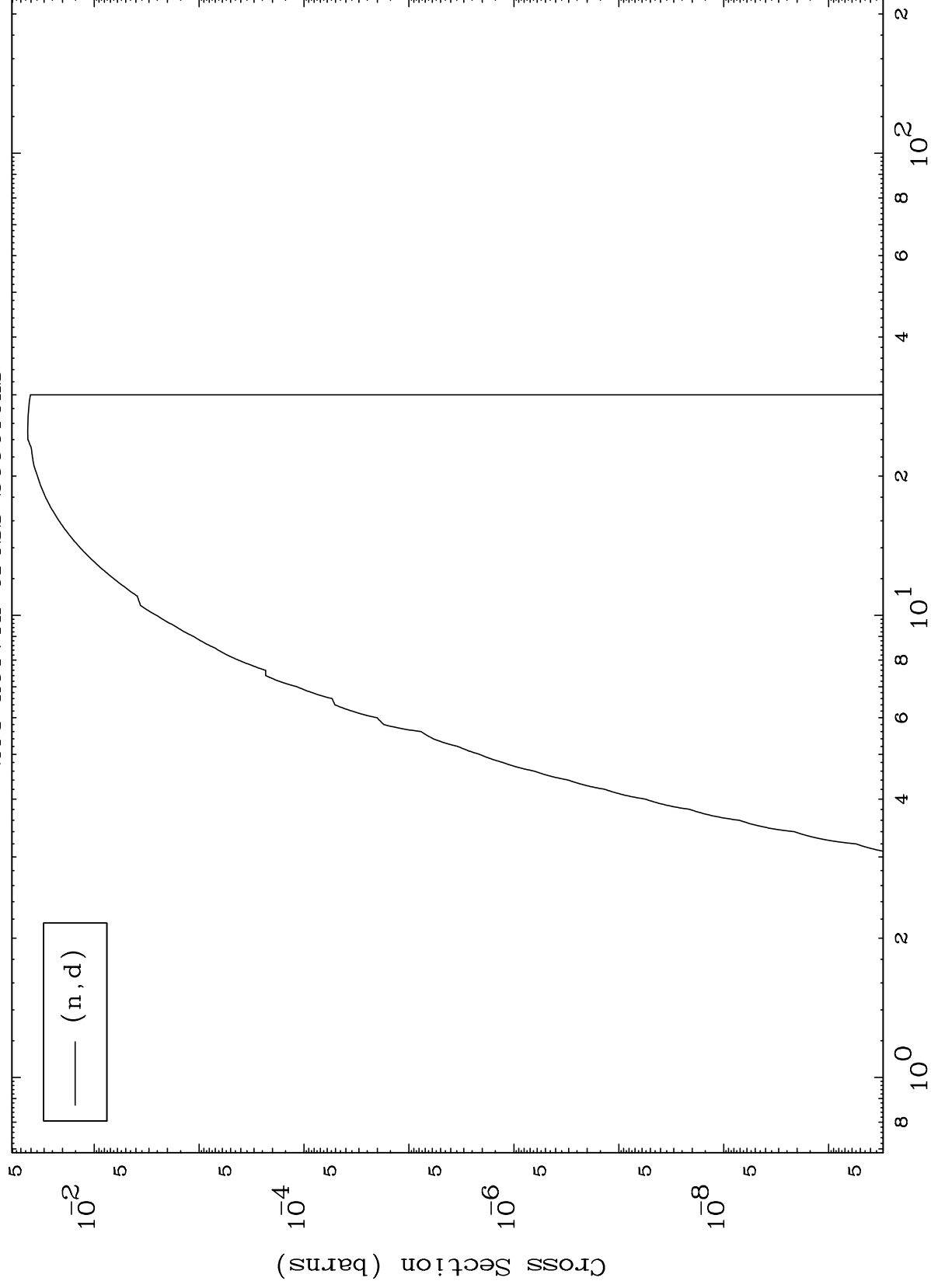


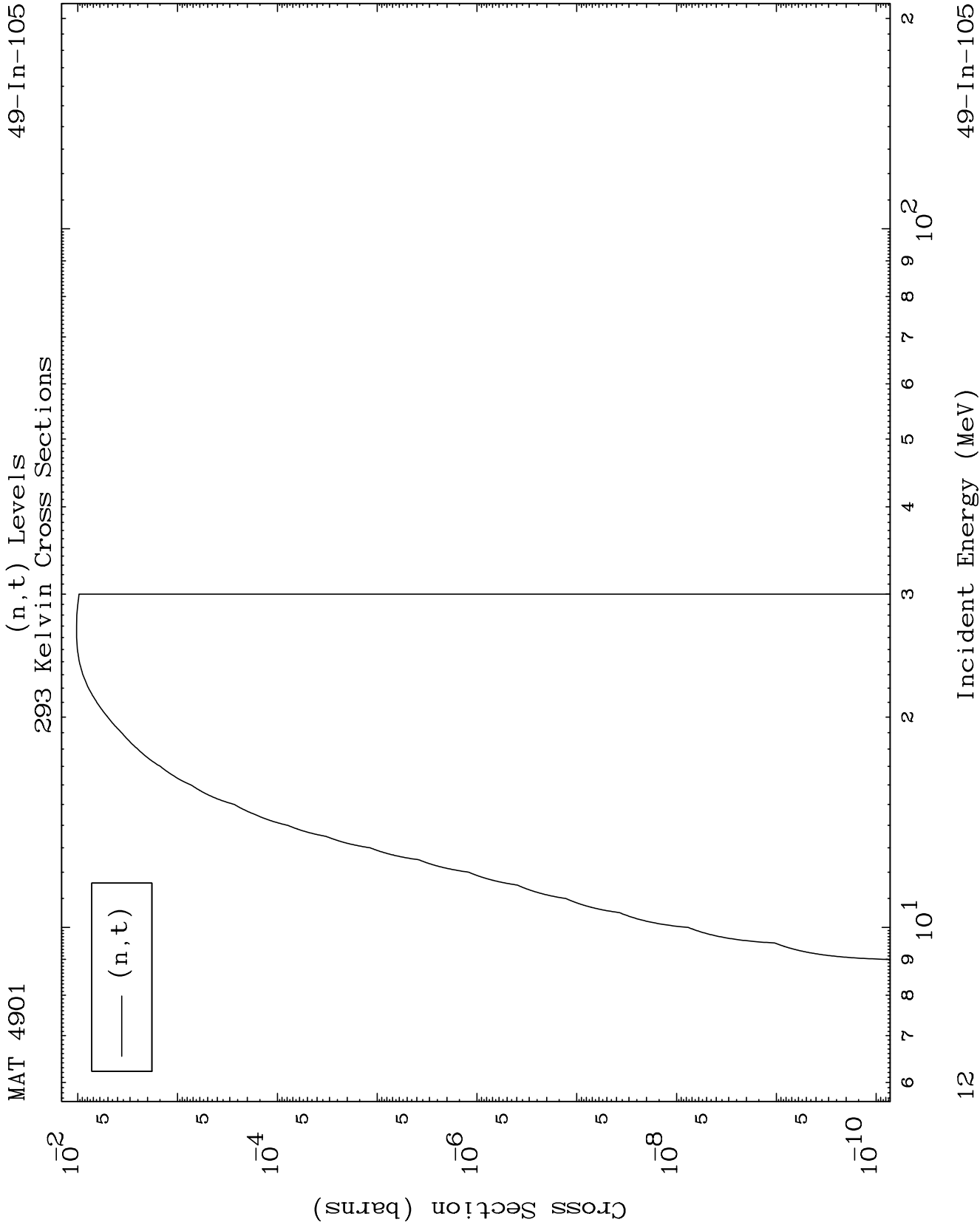


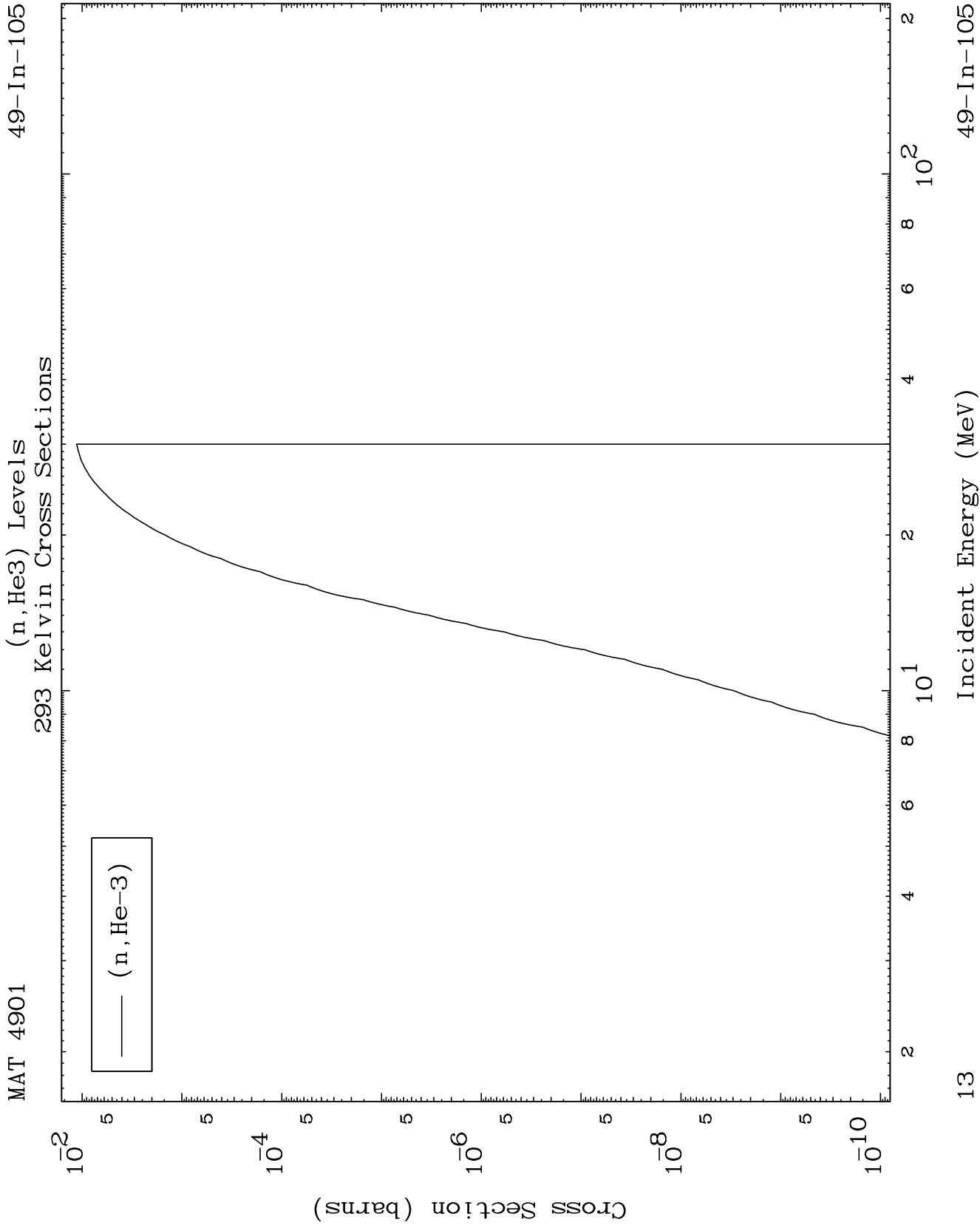
MAT 4901

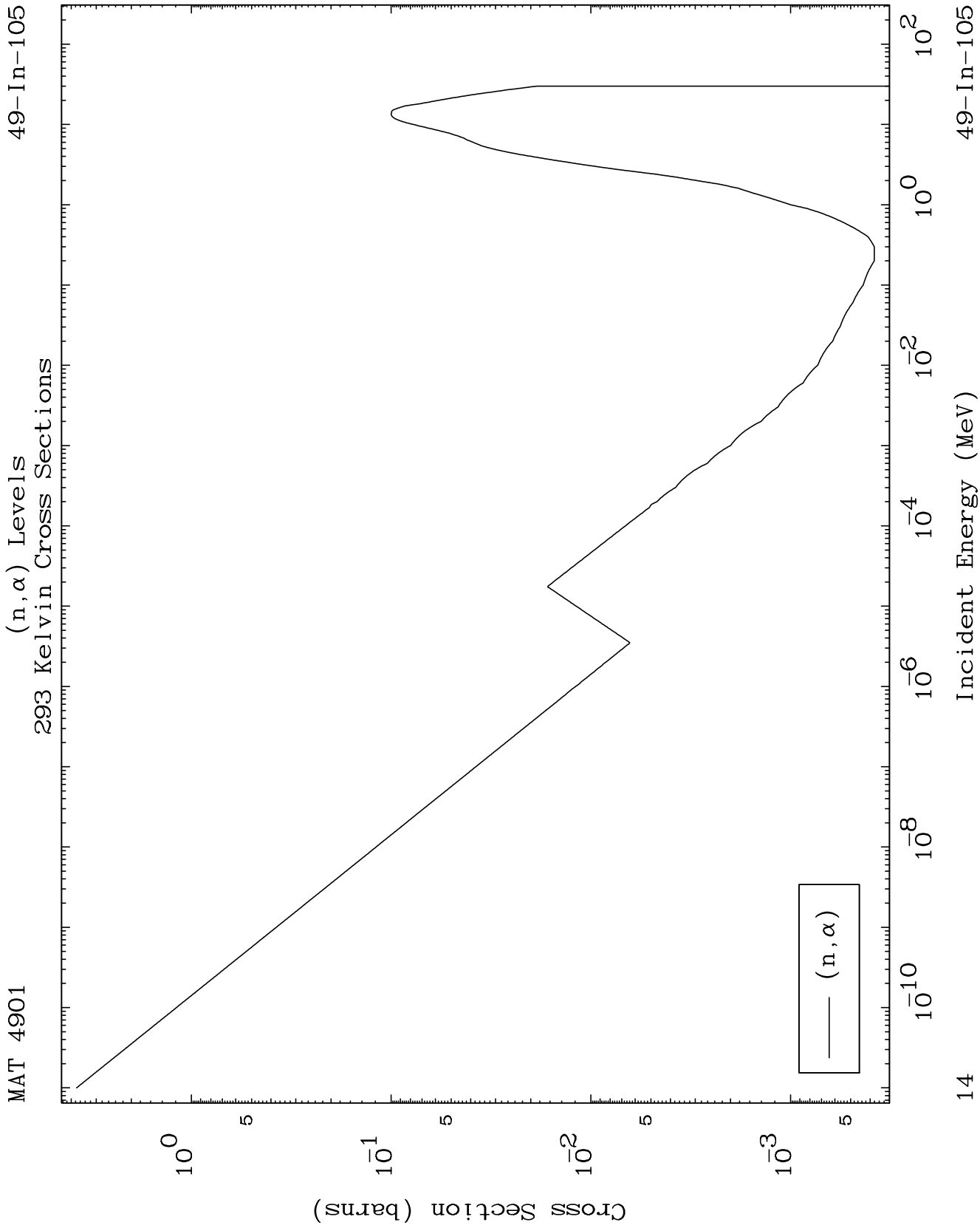
49-In-105

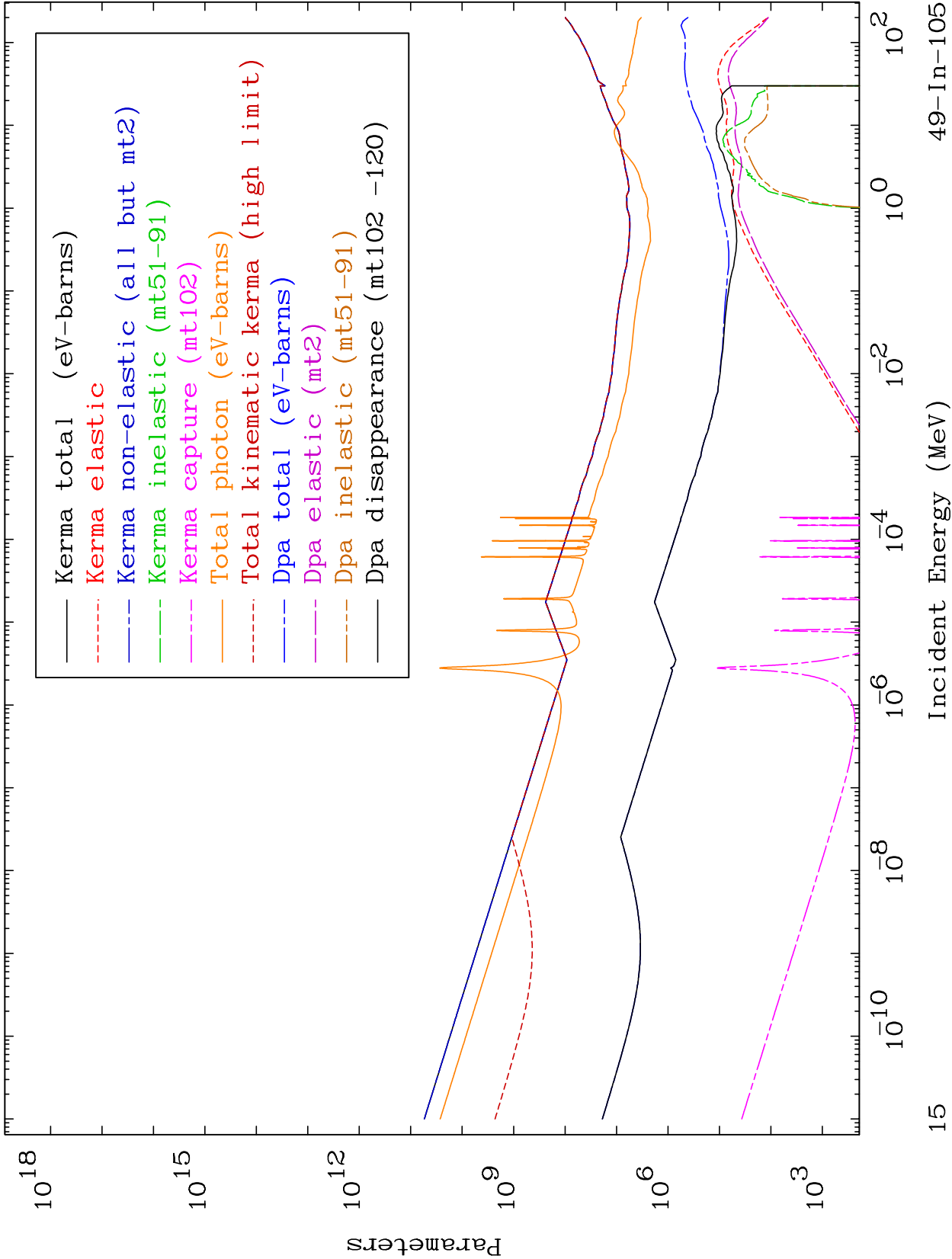
(n,d) Levels
293 Kelvin Cross Sections

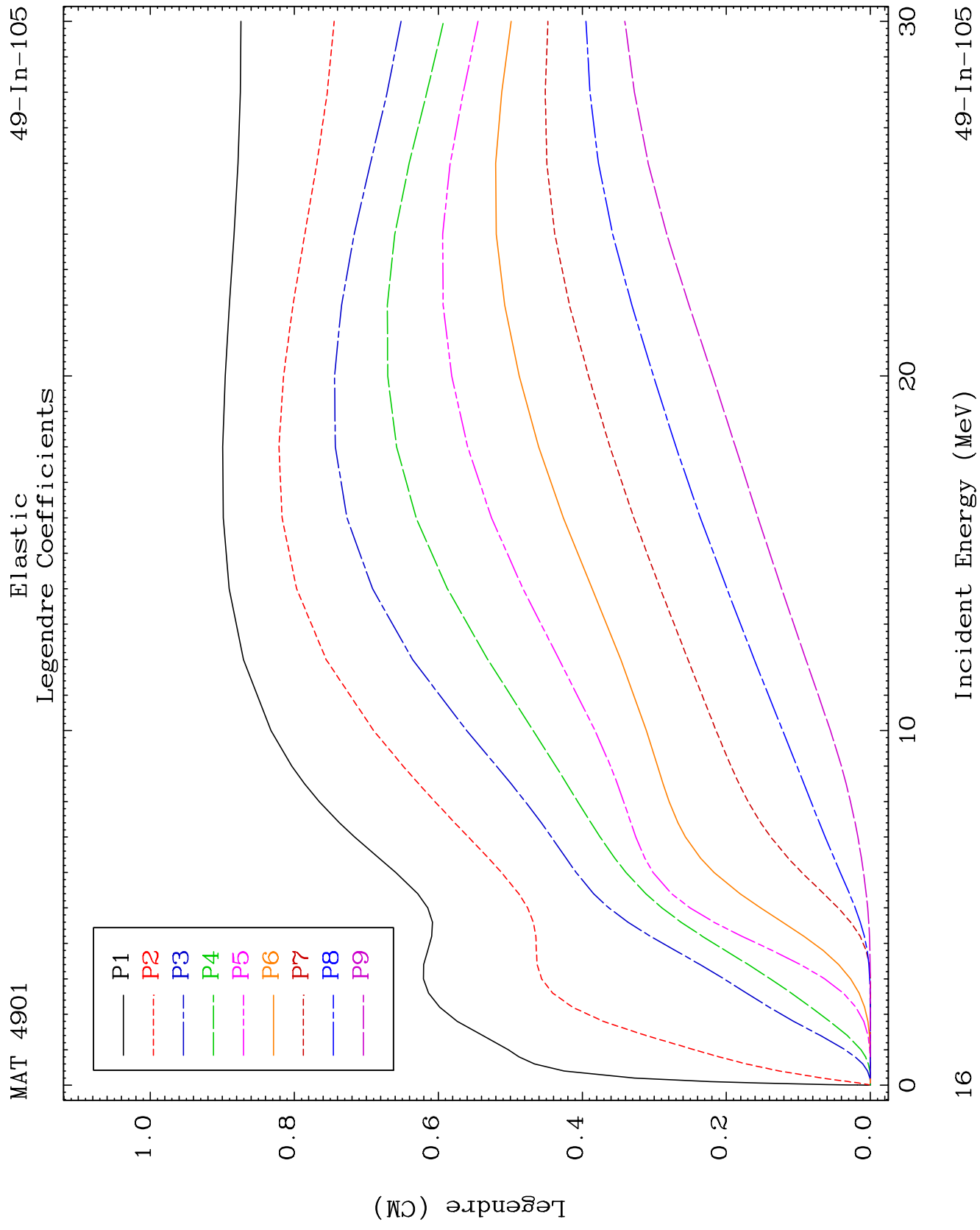


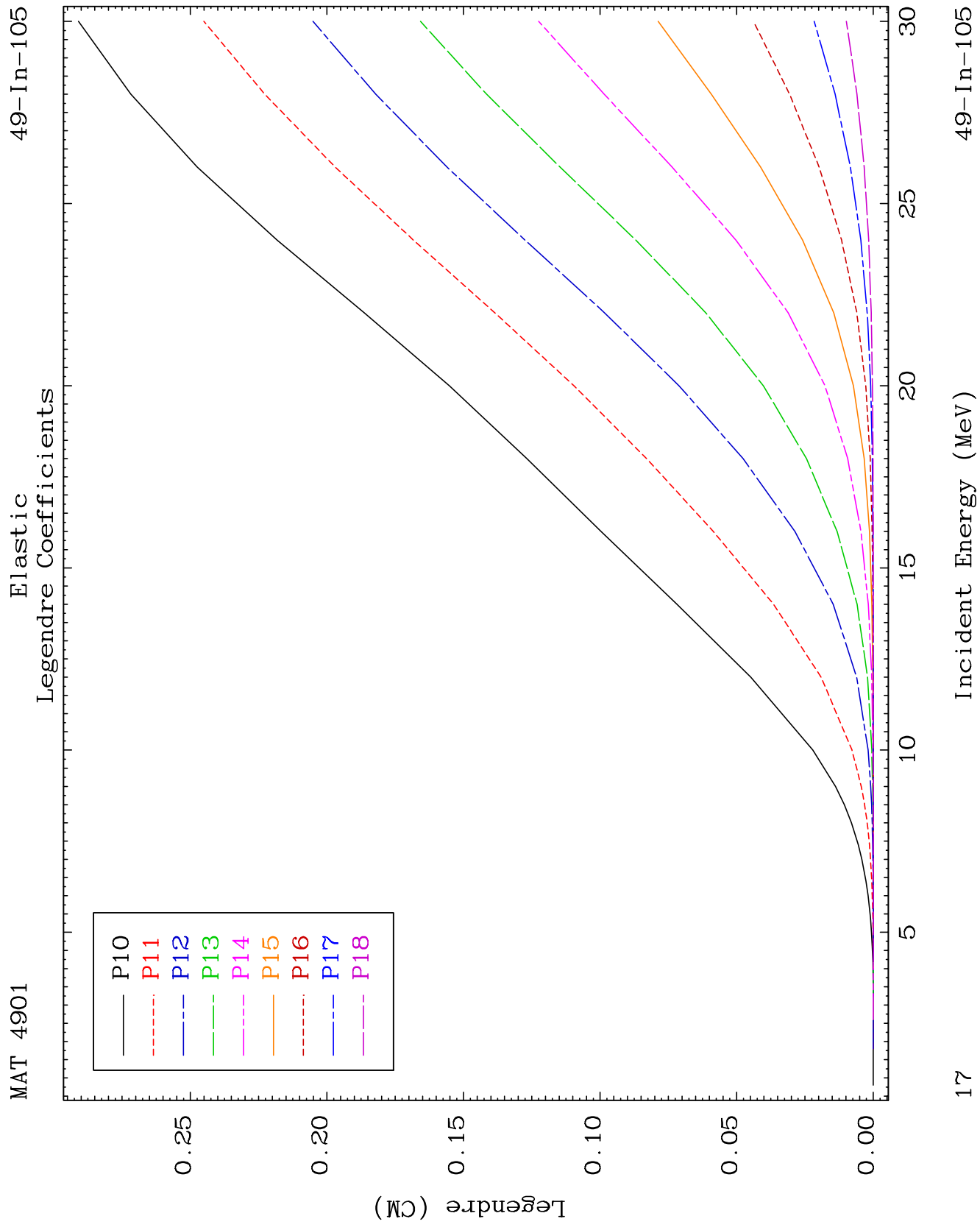


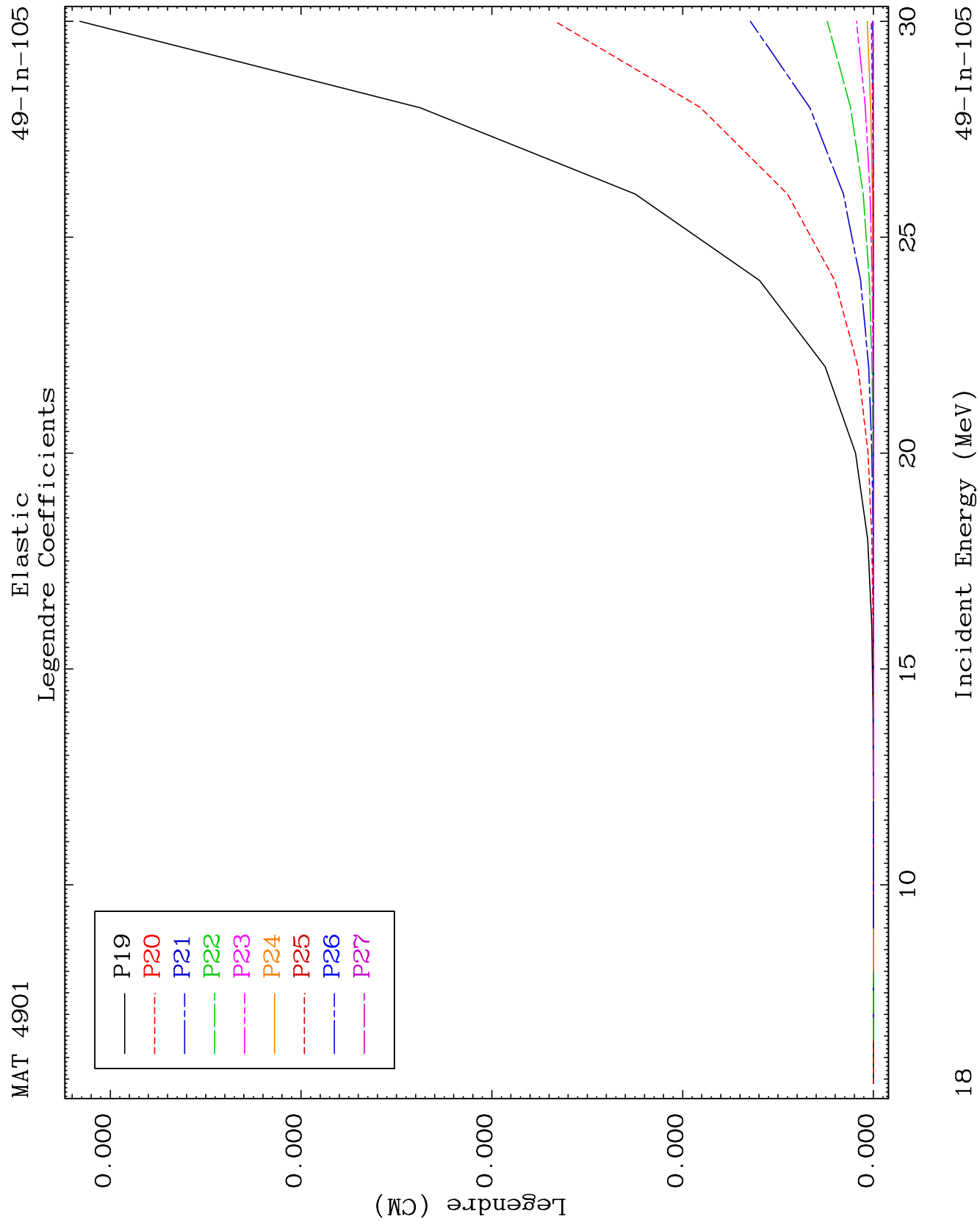


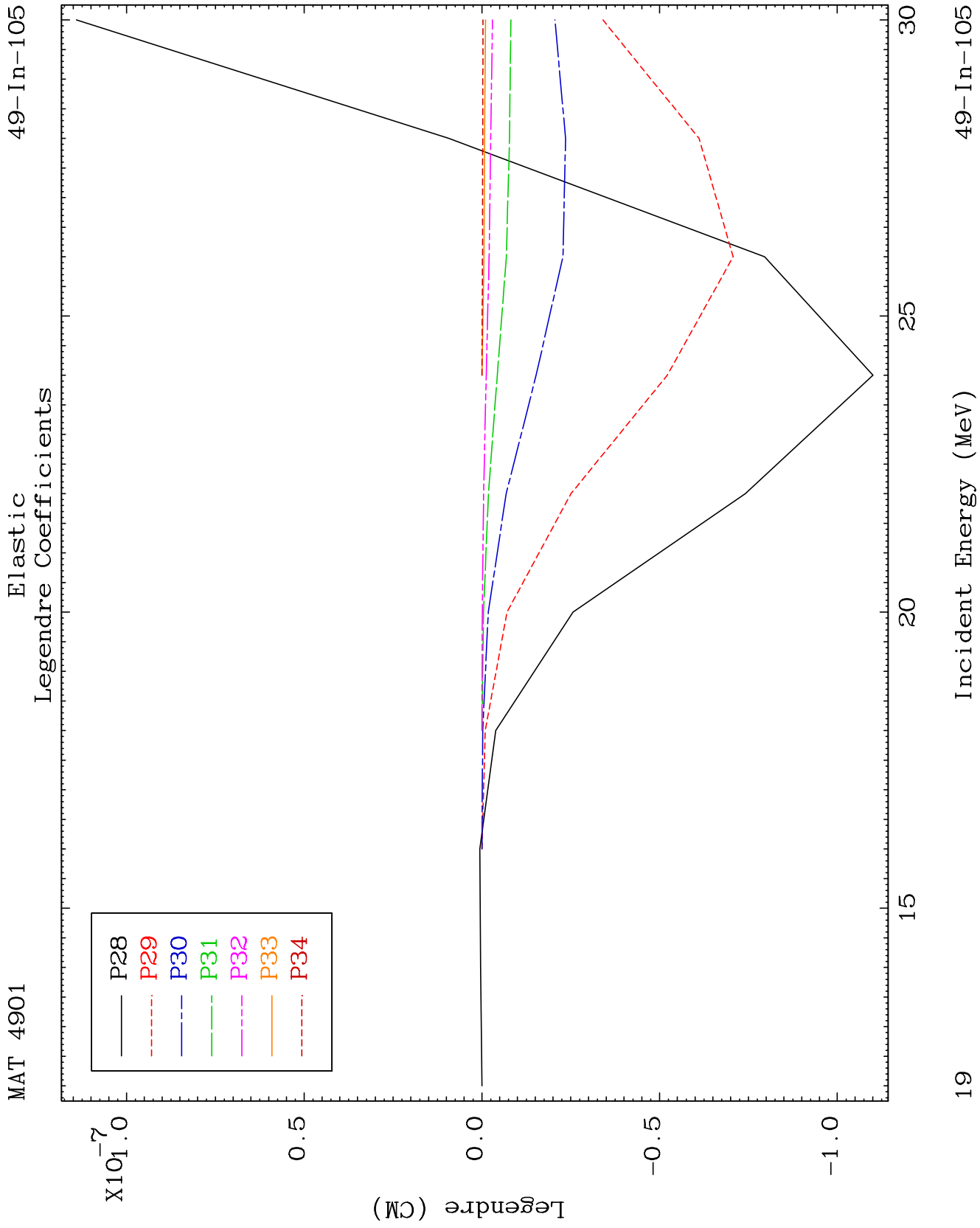








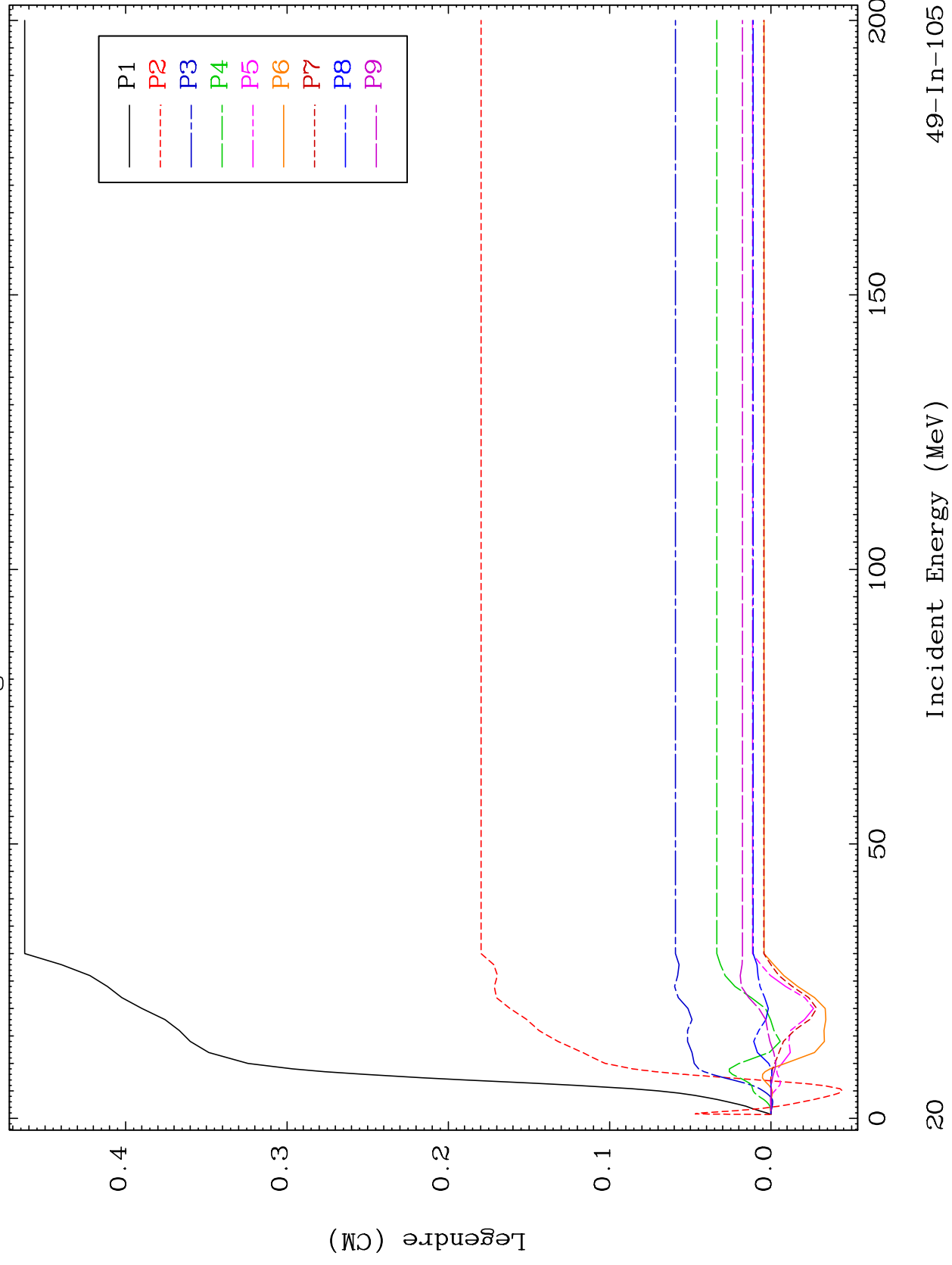


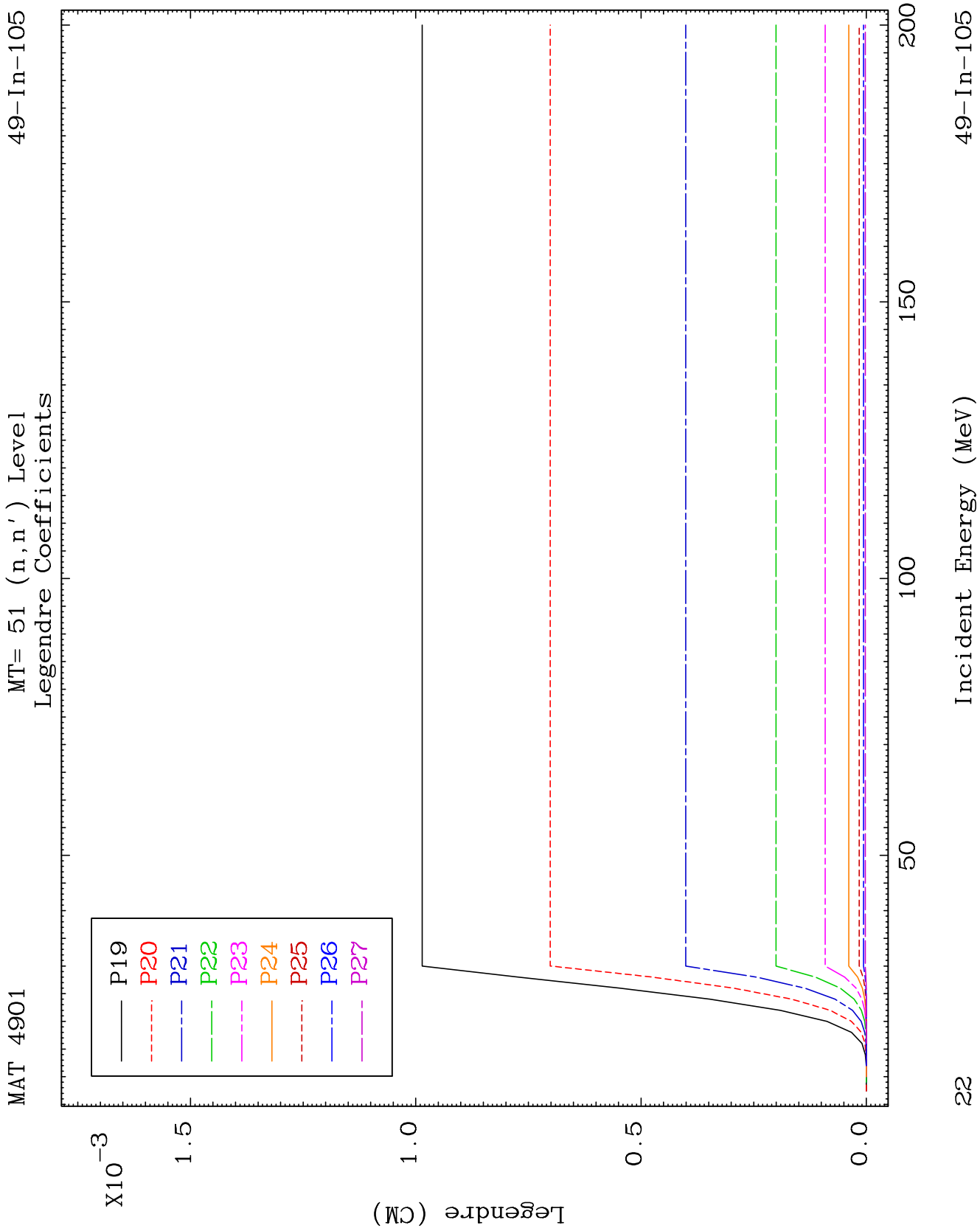


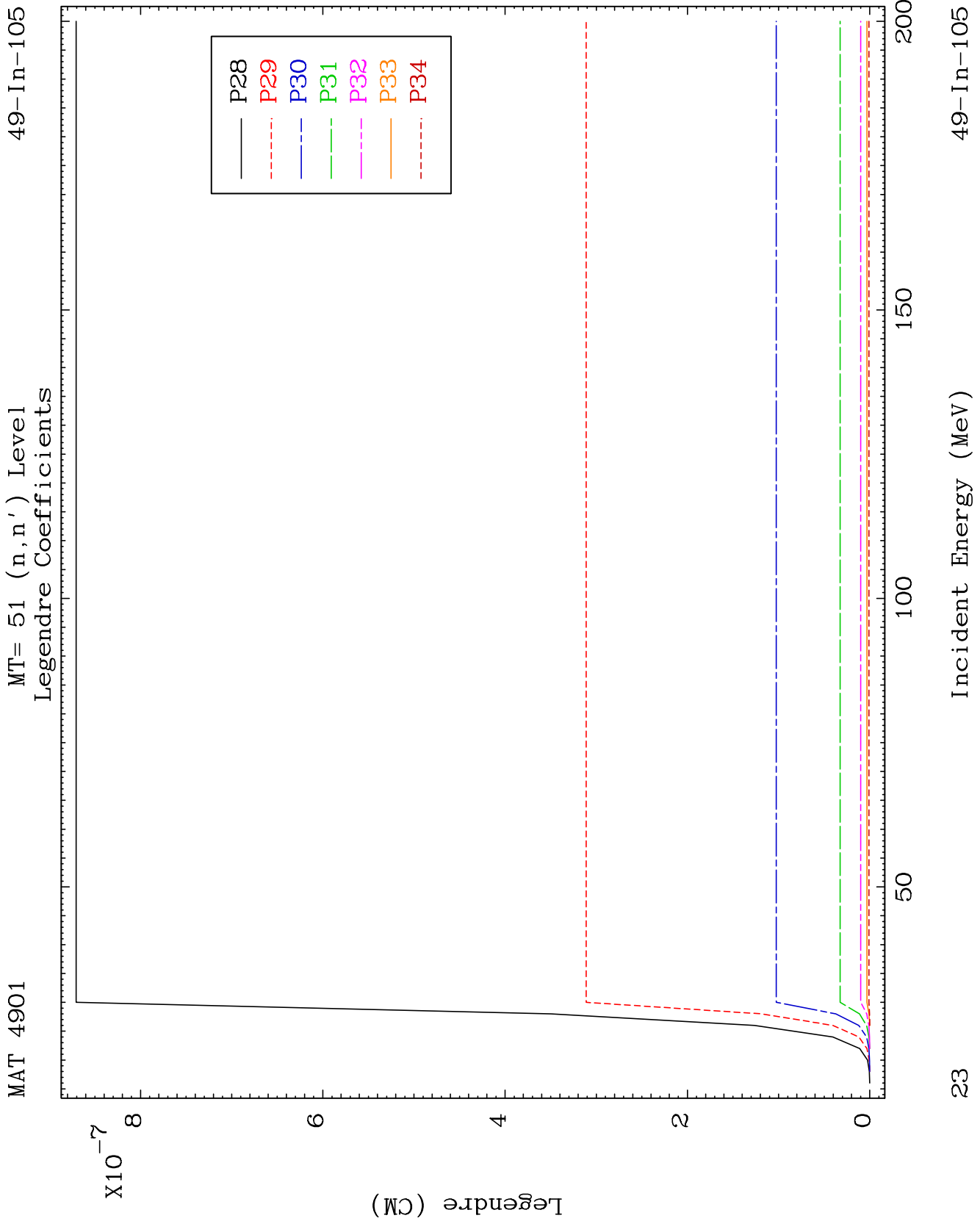
MAT 4901

MT= 51 (n, n') Level
Legendre Coefficients

49-In-105



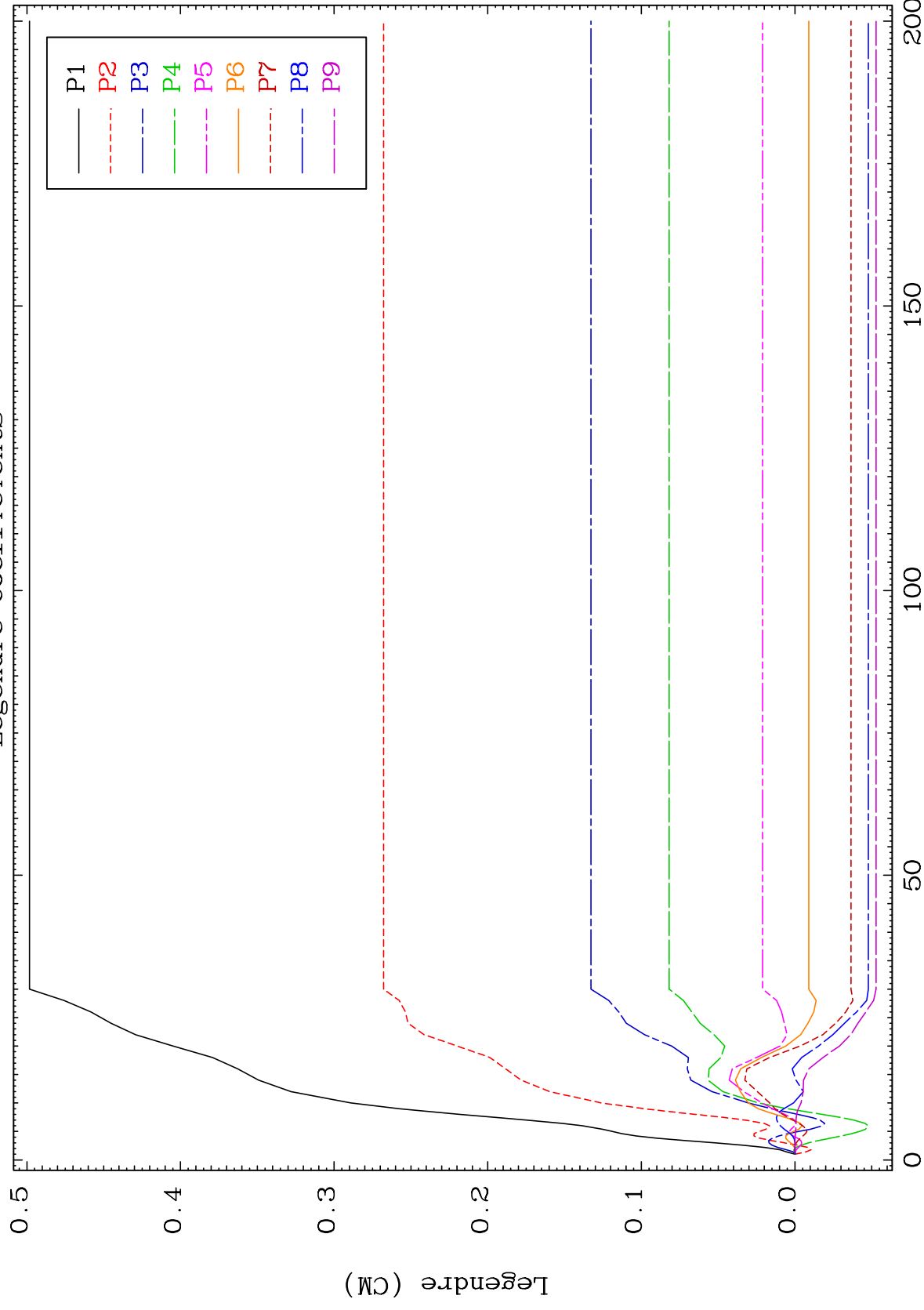




MAT 4901

MT= 52 (n, n') Level
Legendre Coefficients

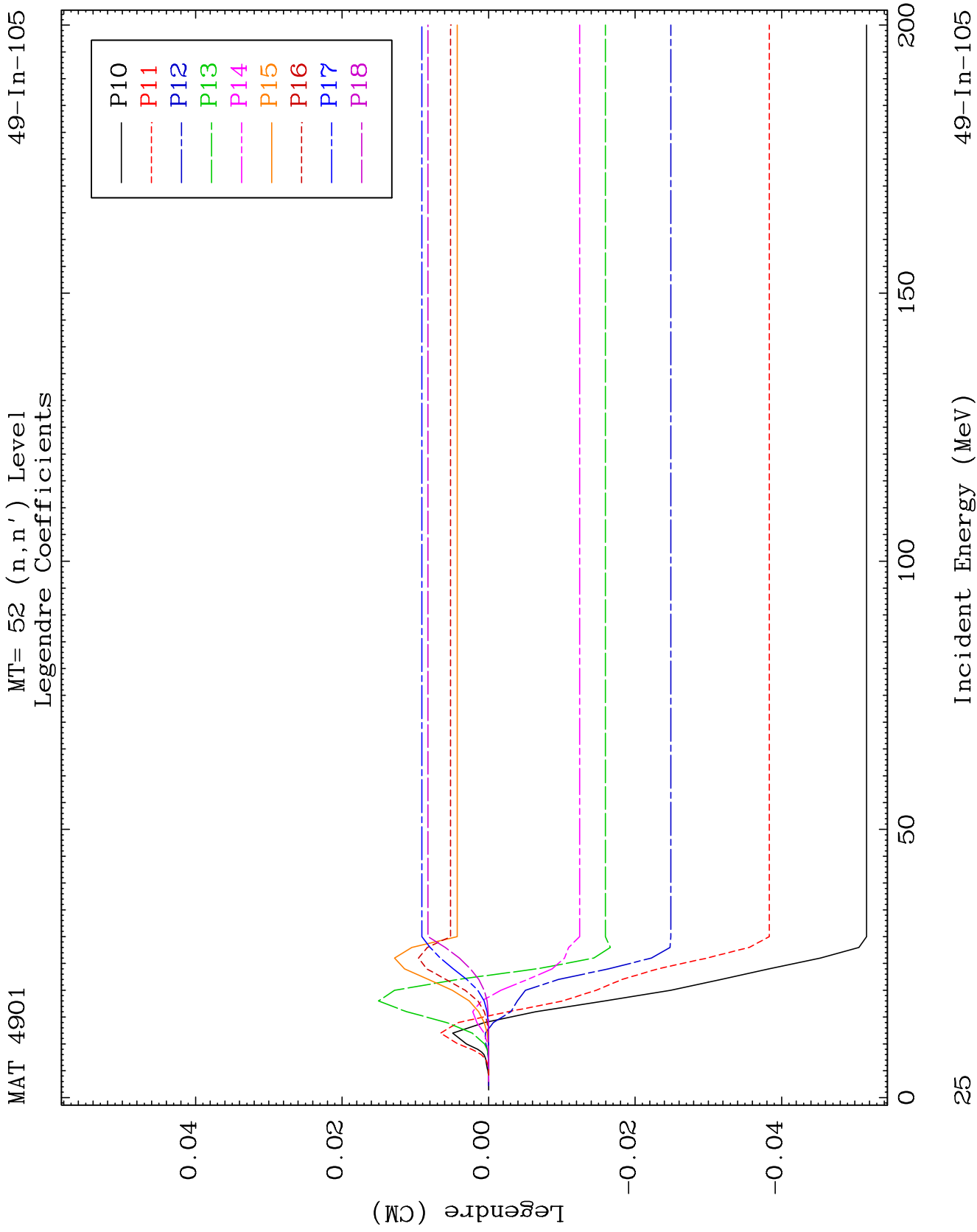
49-In-105

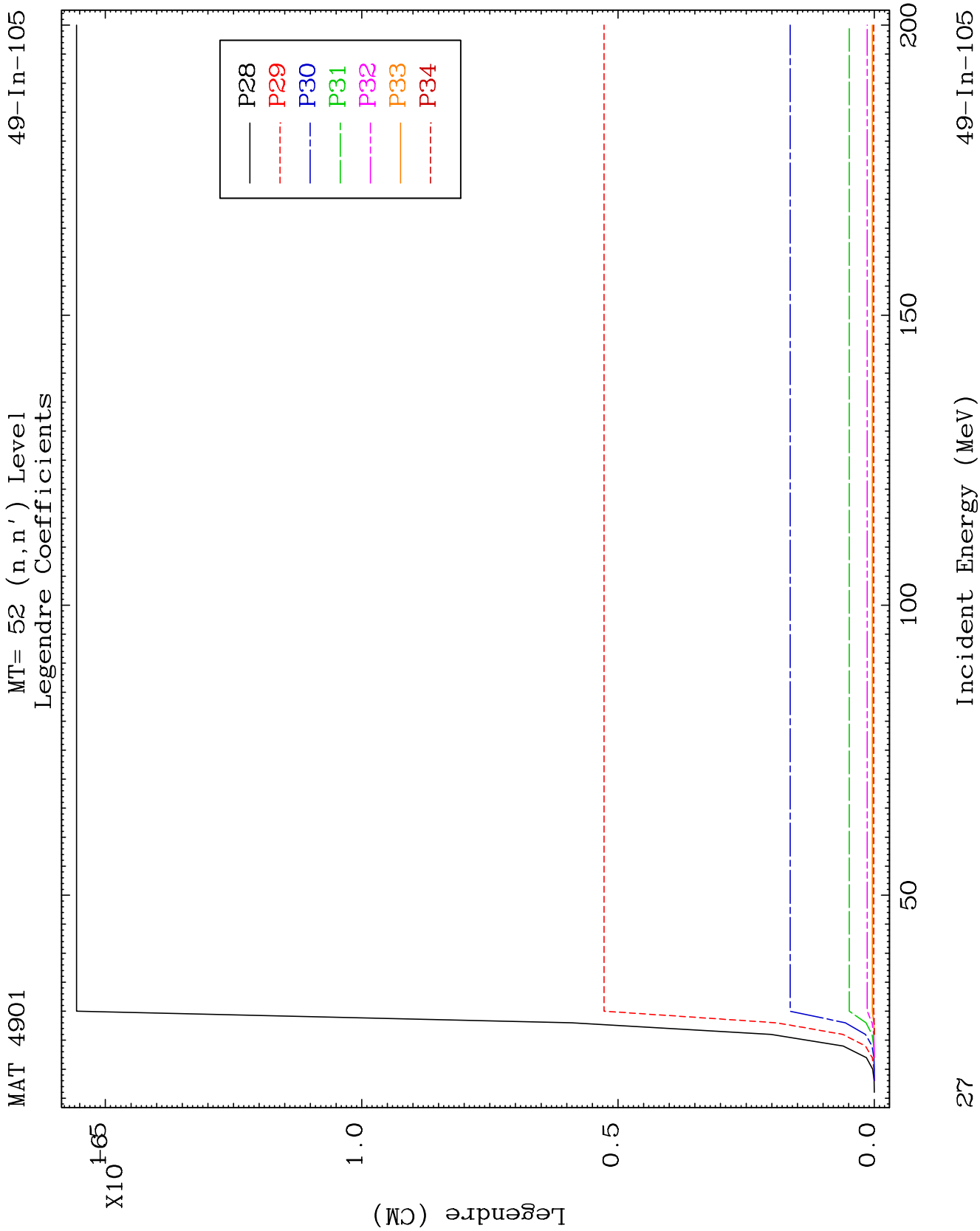


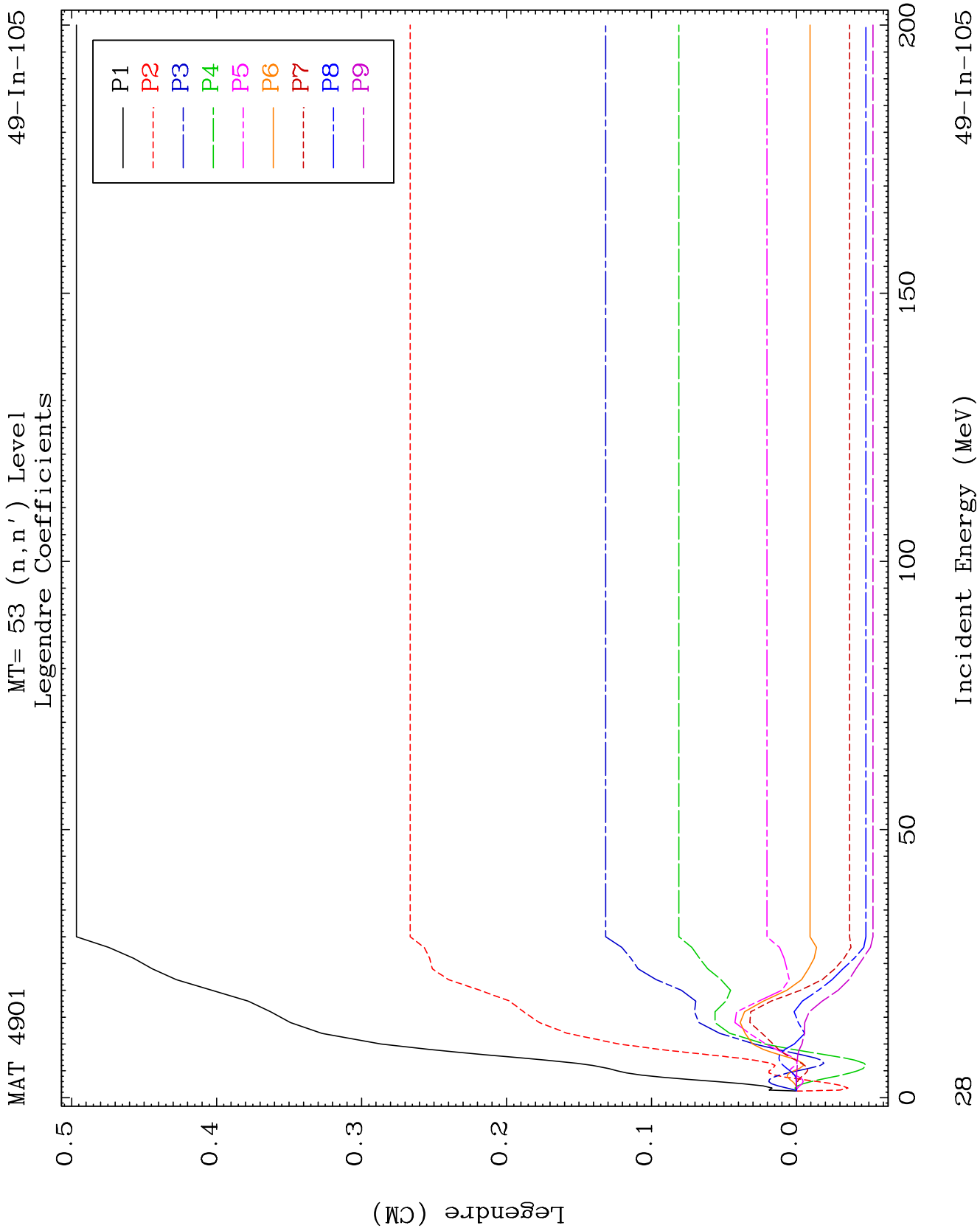
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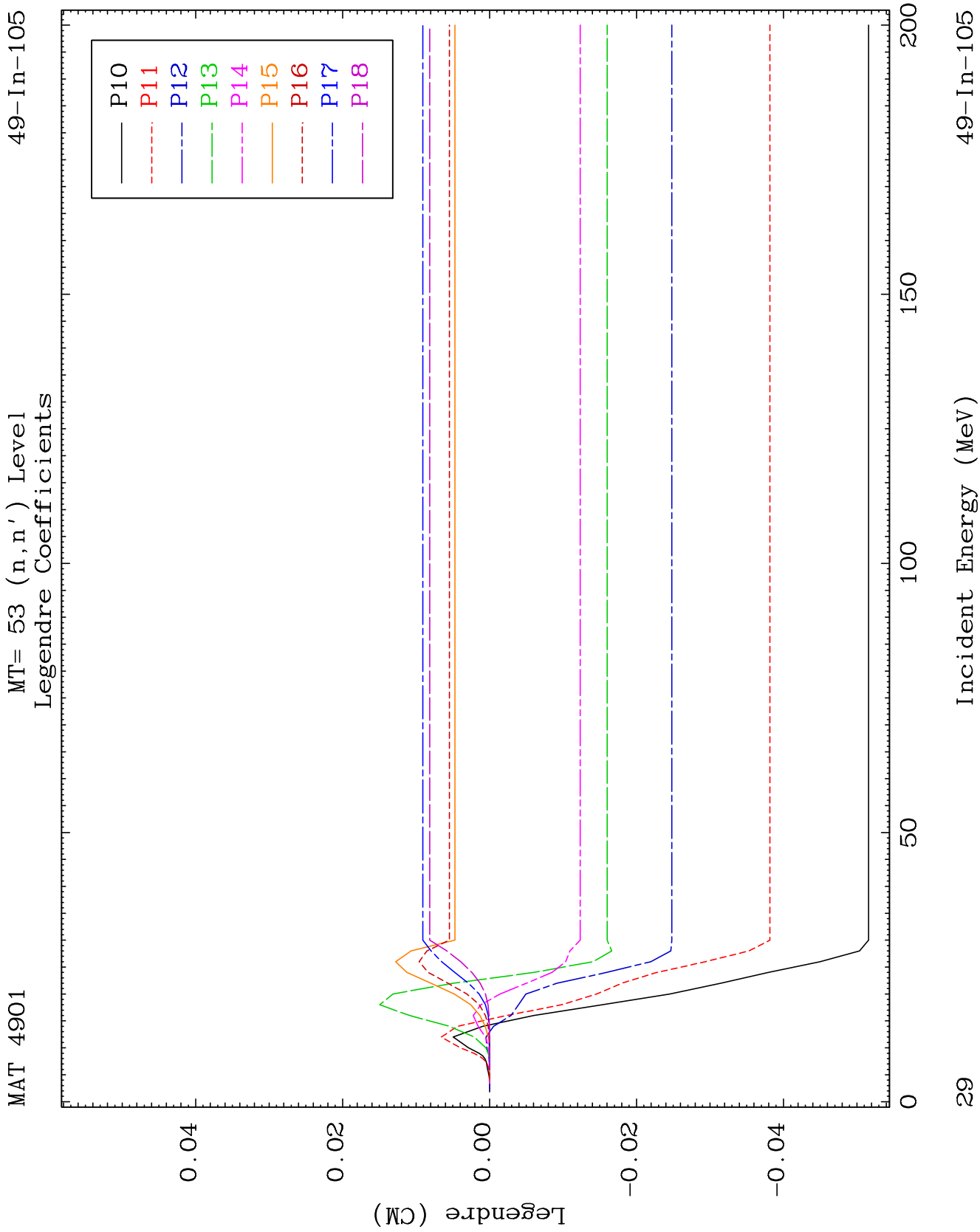
Incident Energy (MeV)

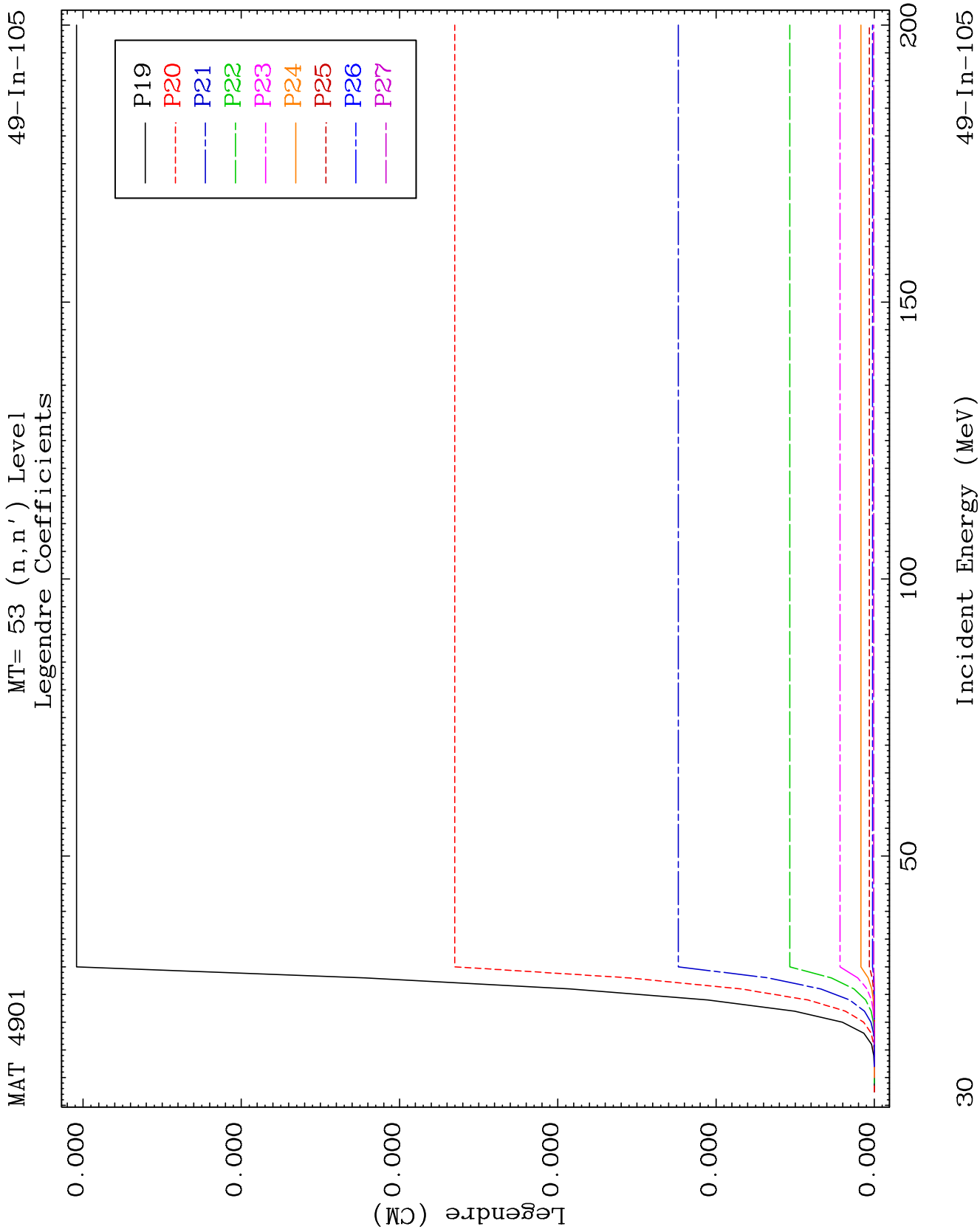
49-In-105

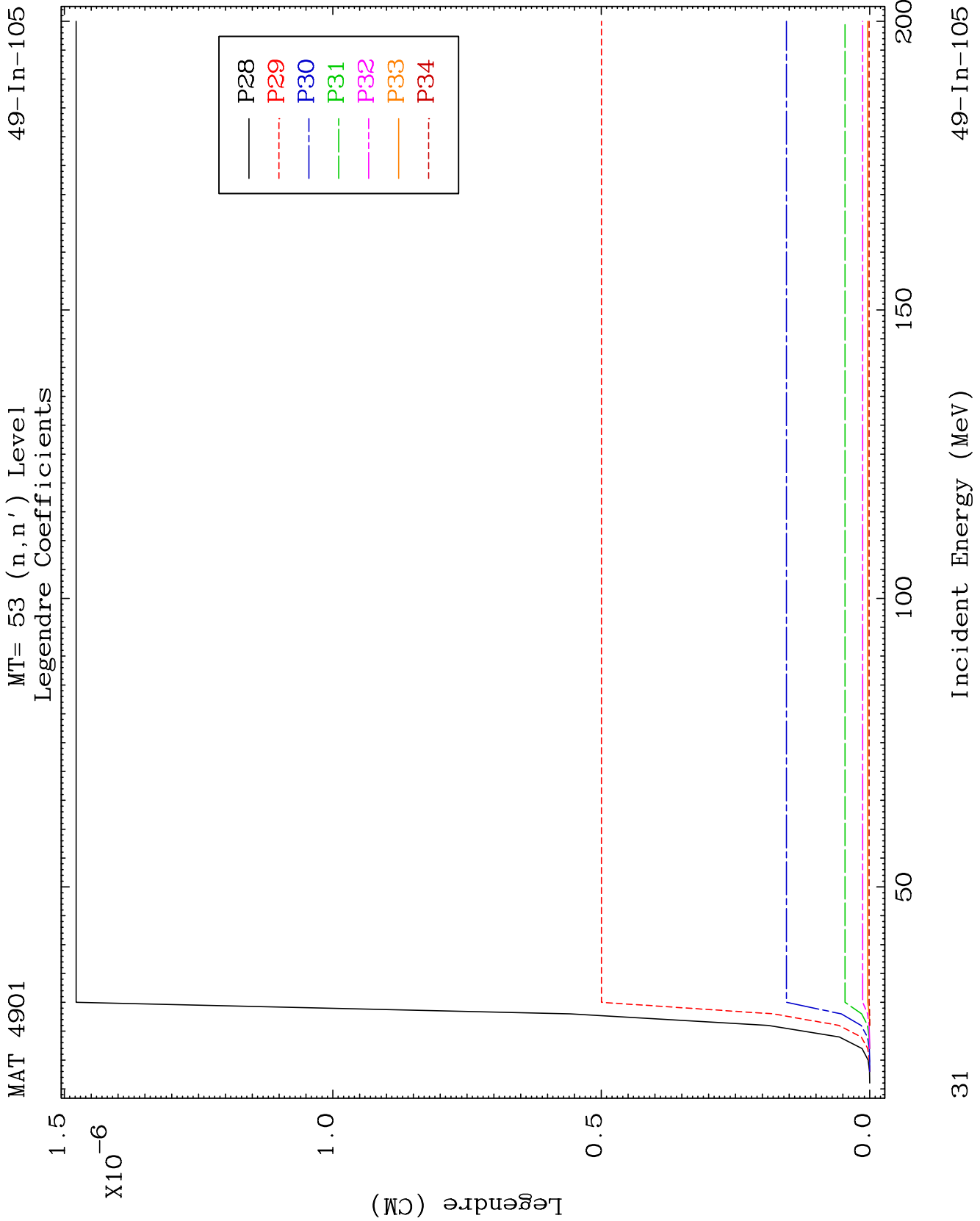








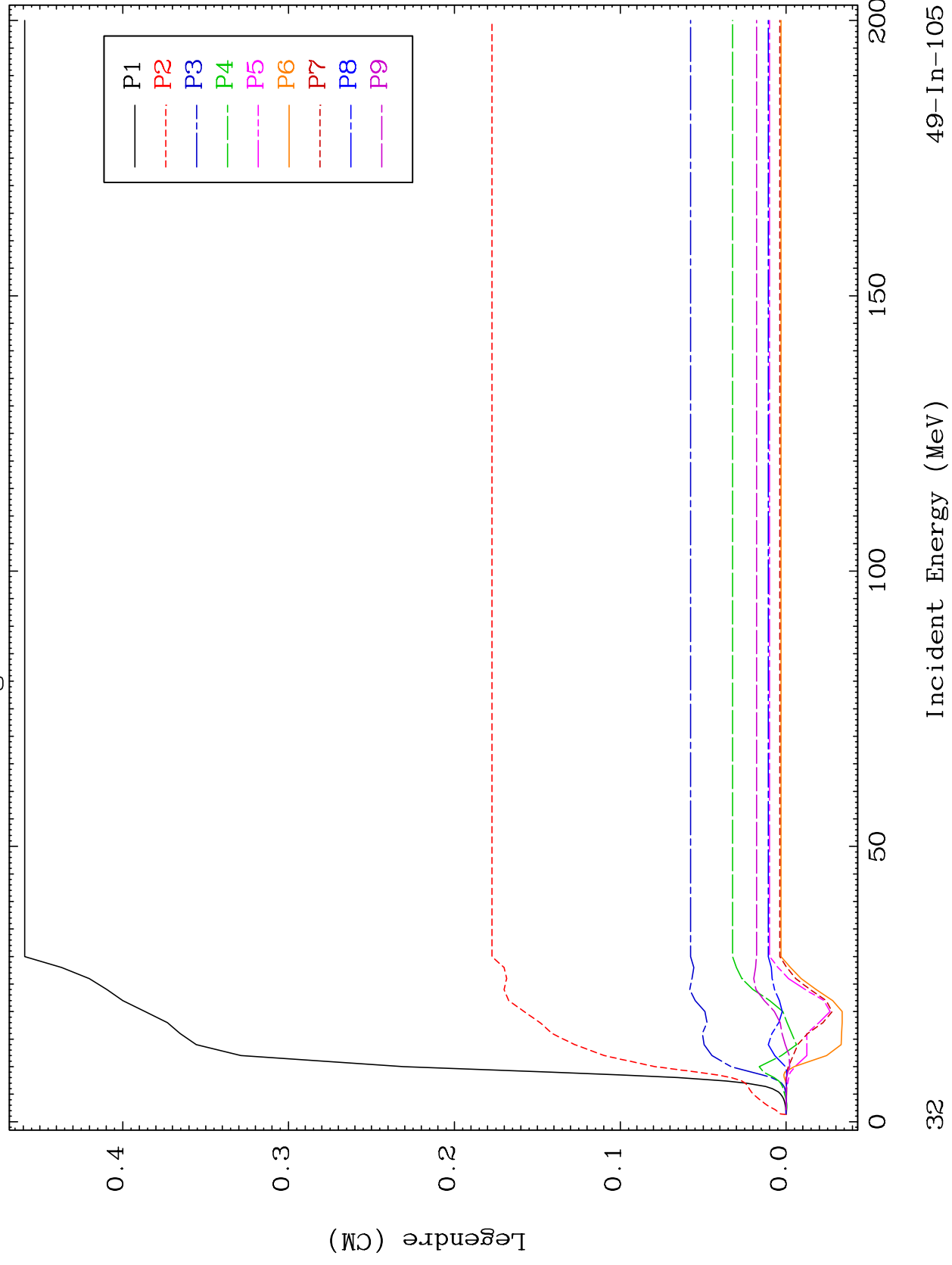




MAT 4901

MT= 54 (n, n') Level
Legendre Coefficients

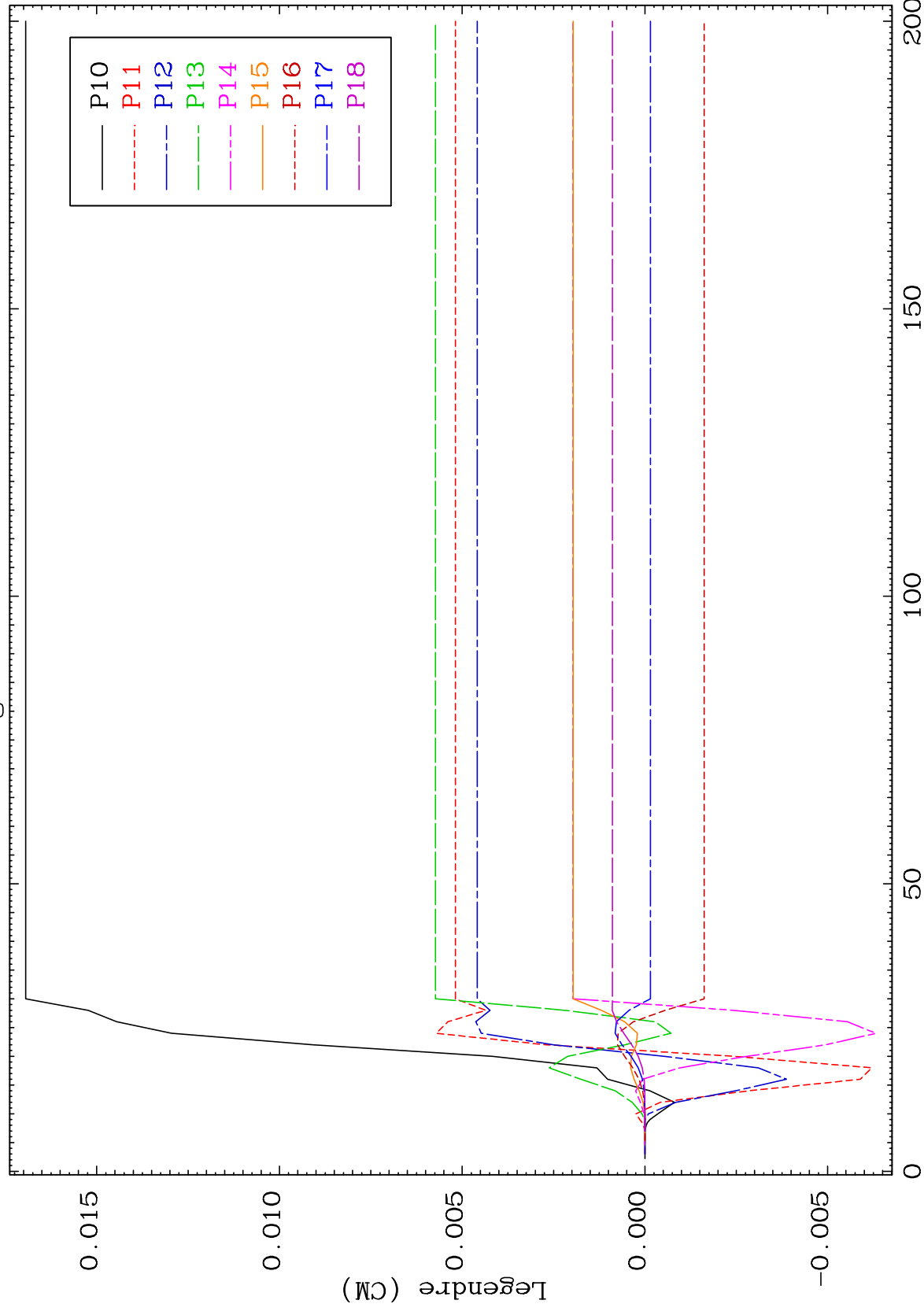
49-In-105



MAT 4901

MT= 54 (n, n') Level
Legendre Coefficients

49-In-105



33

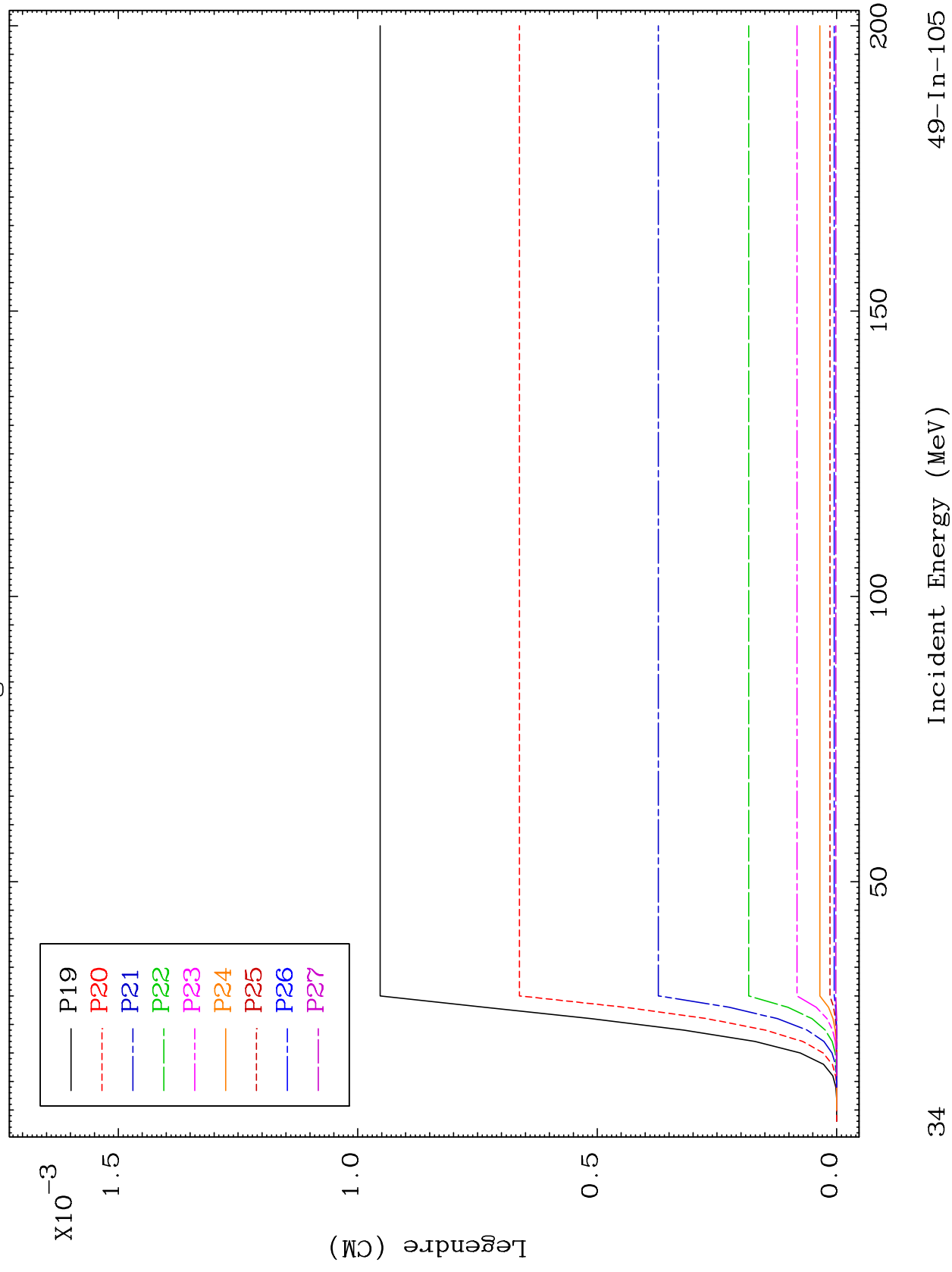
Incident Energy (MeV)

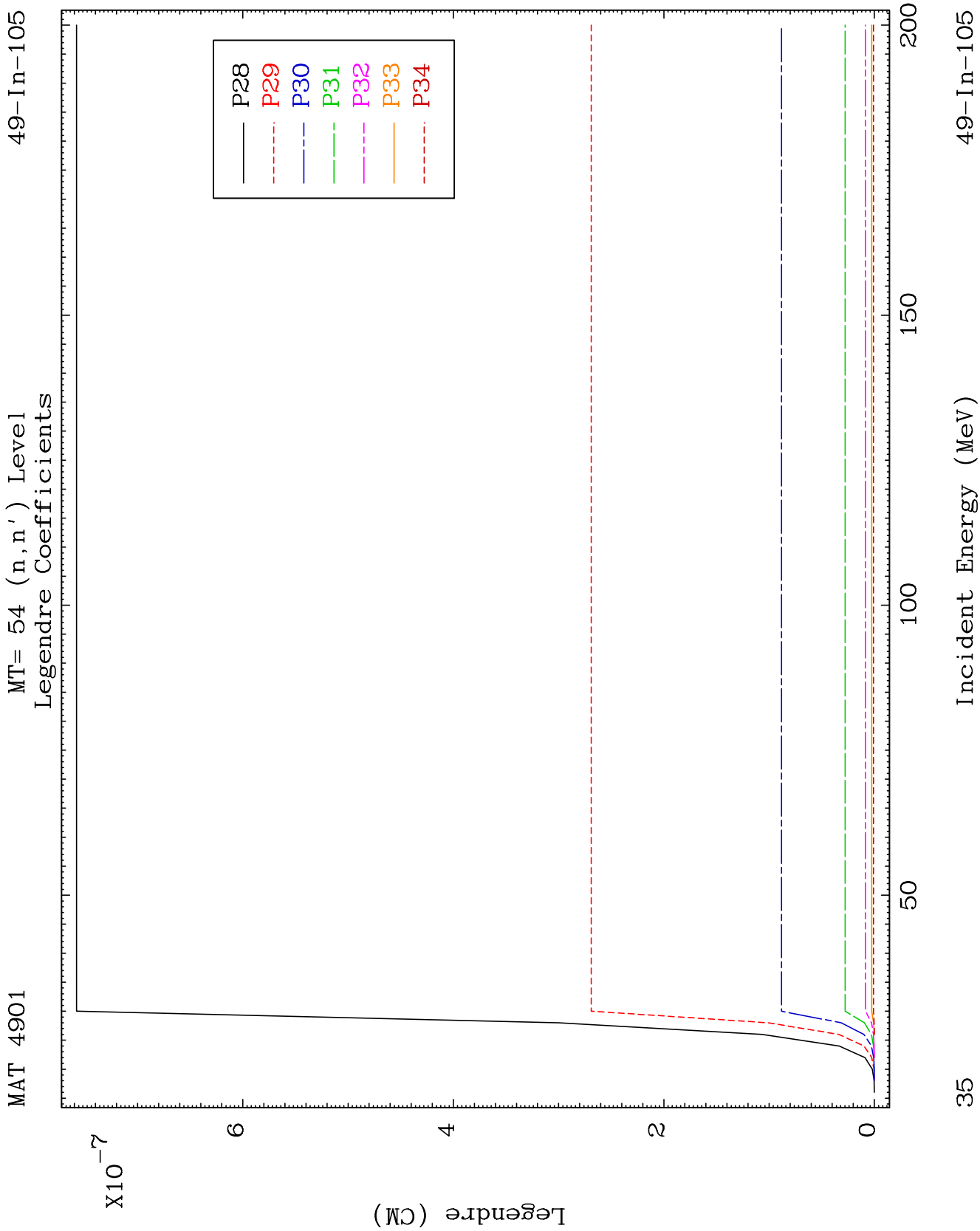
49-In-105

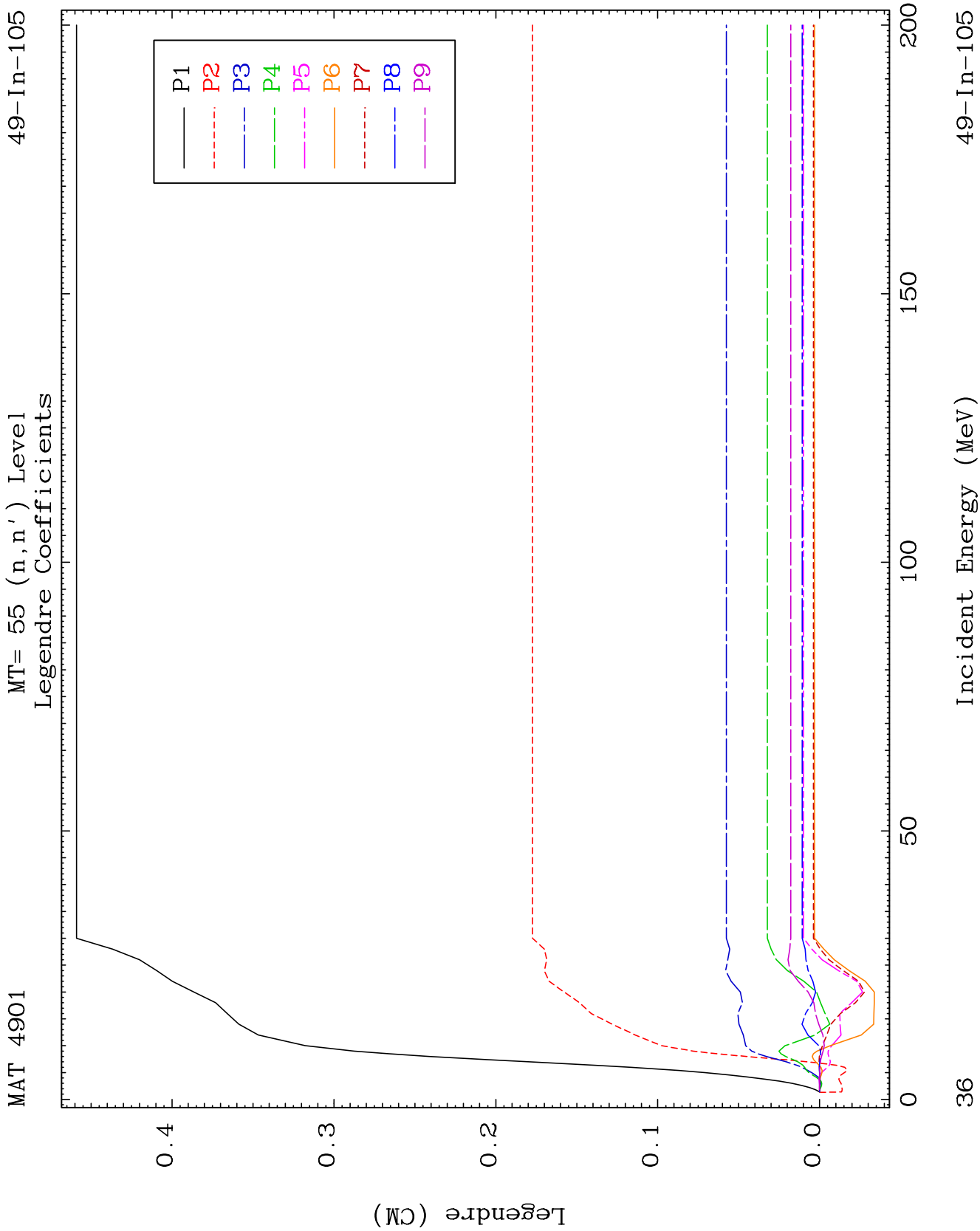
MAT 4901

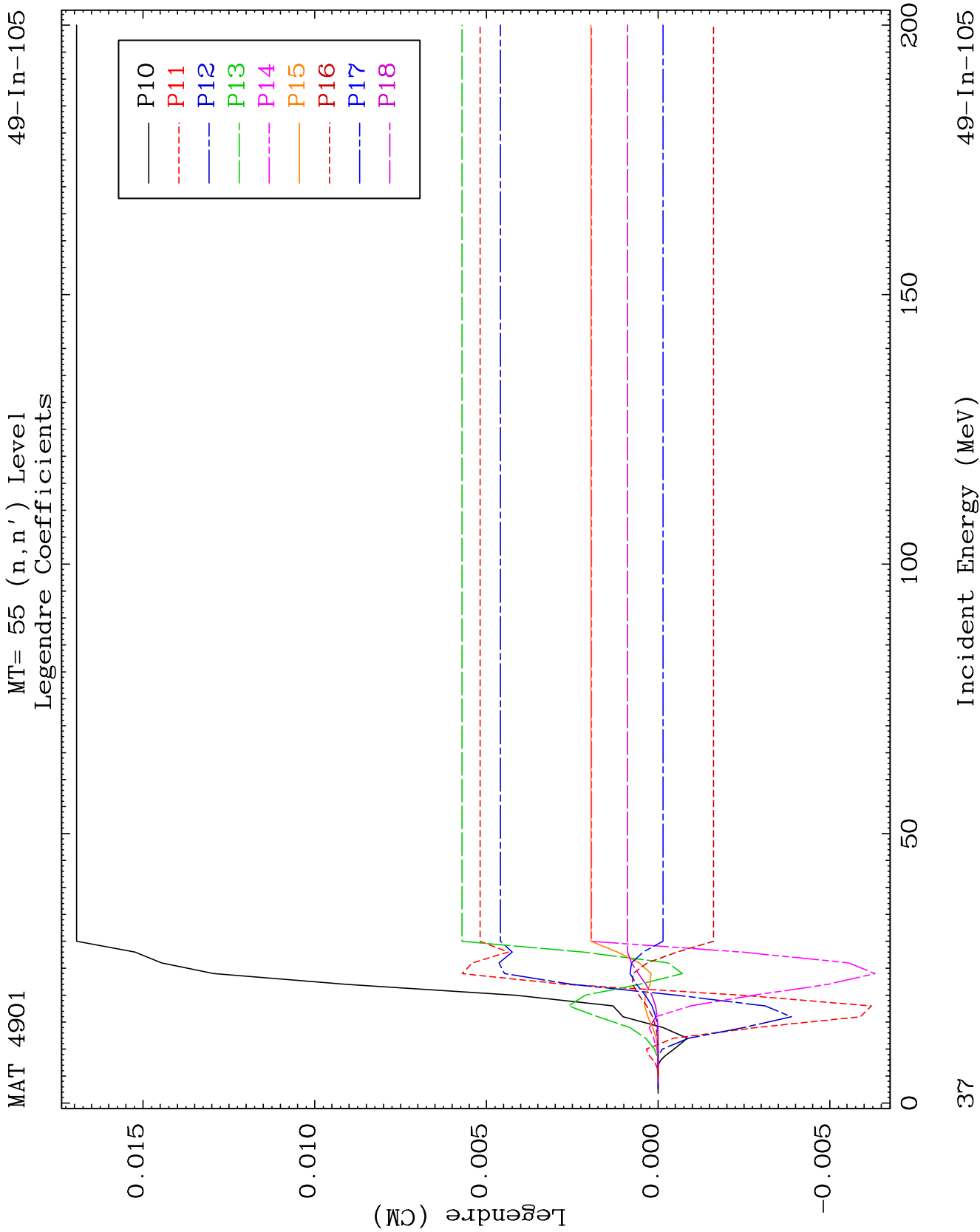
MT= 54 (n, n') Level
Legendre Coefficients

49-In-105





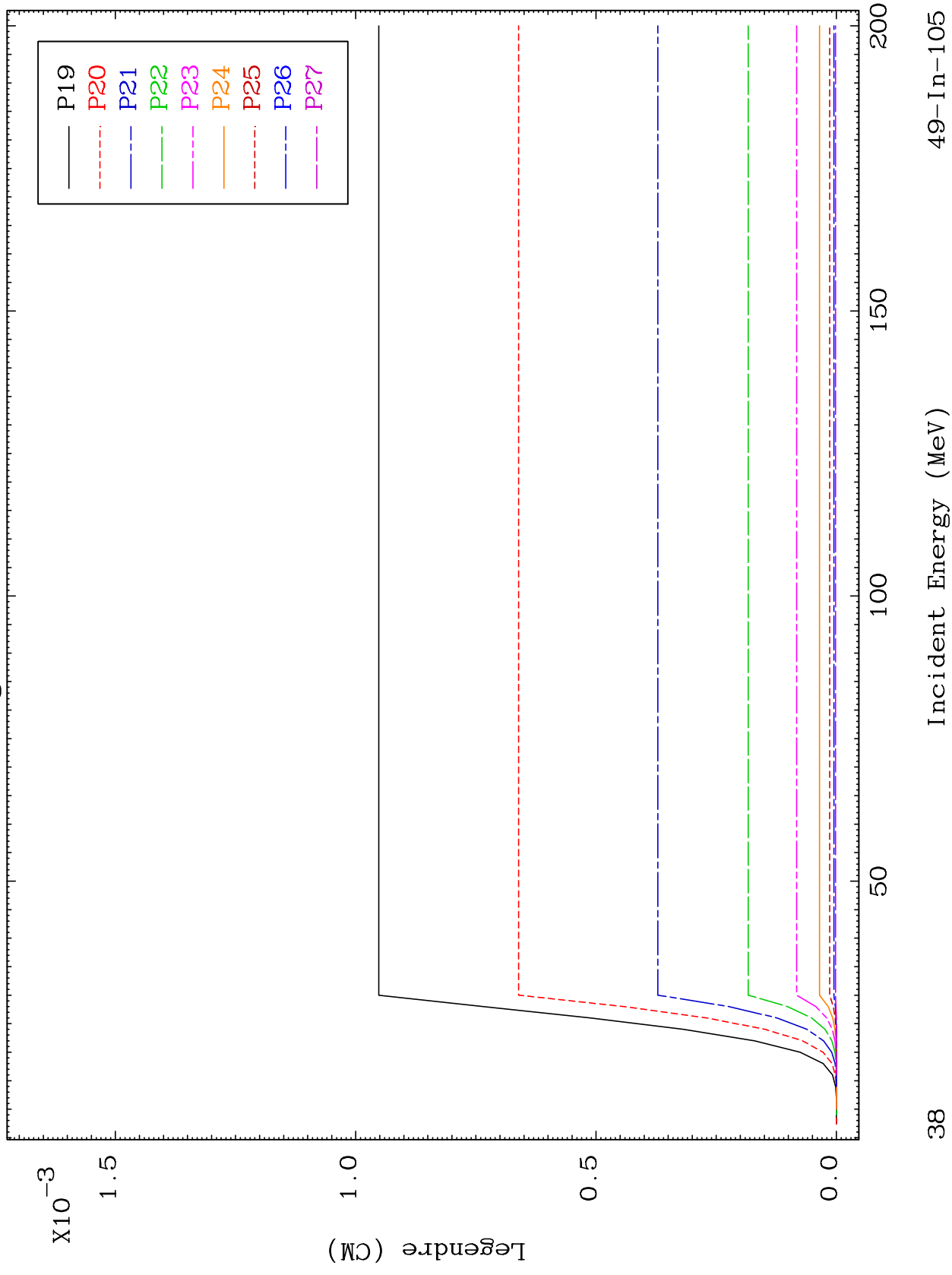


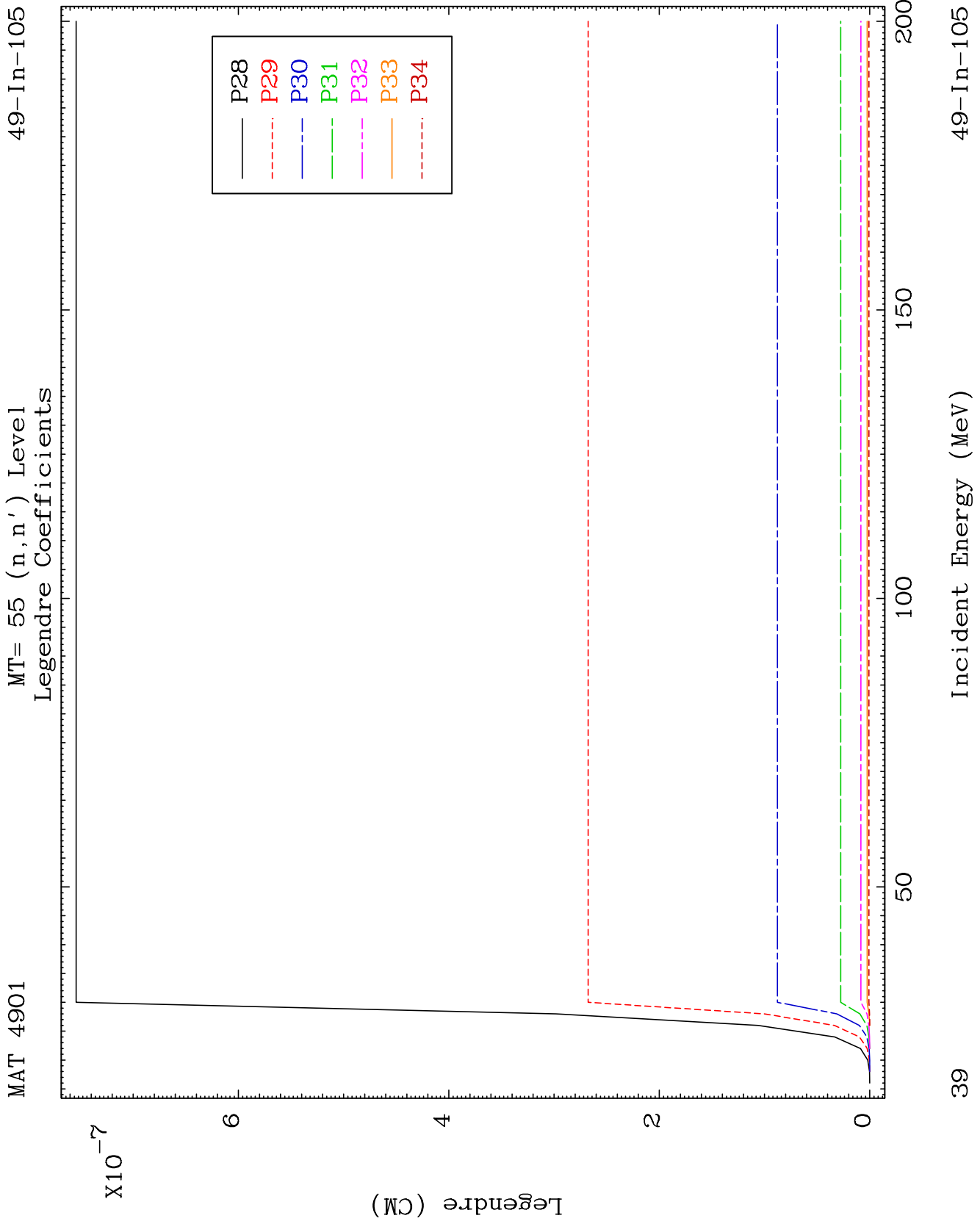


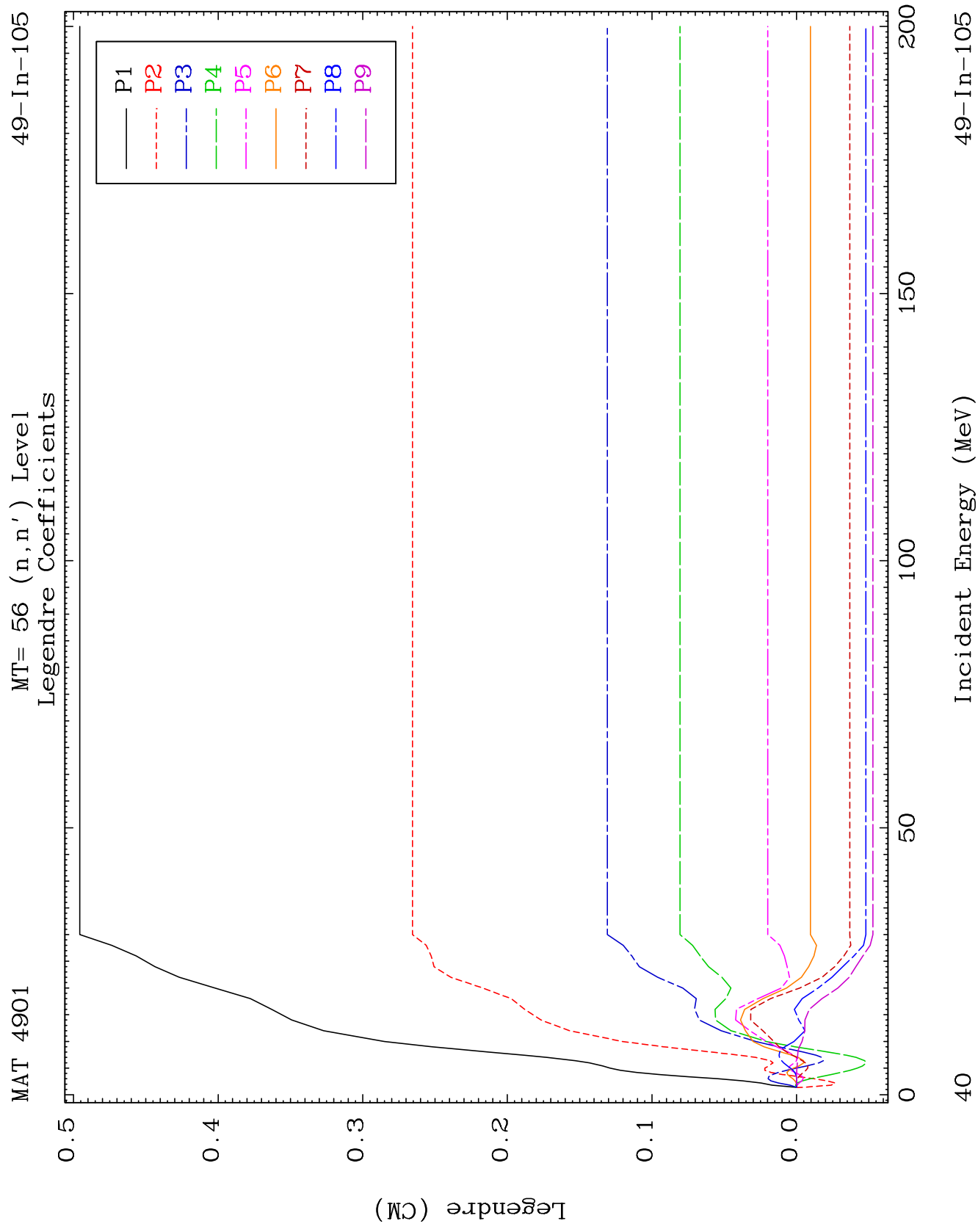
MAT 4901

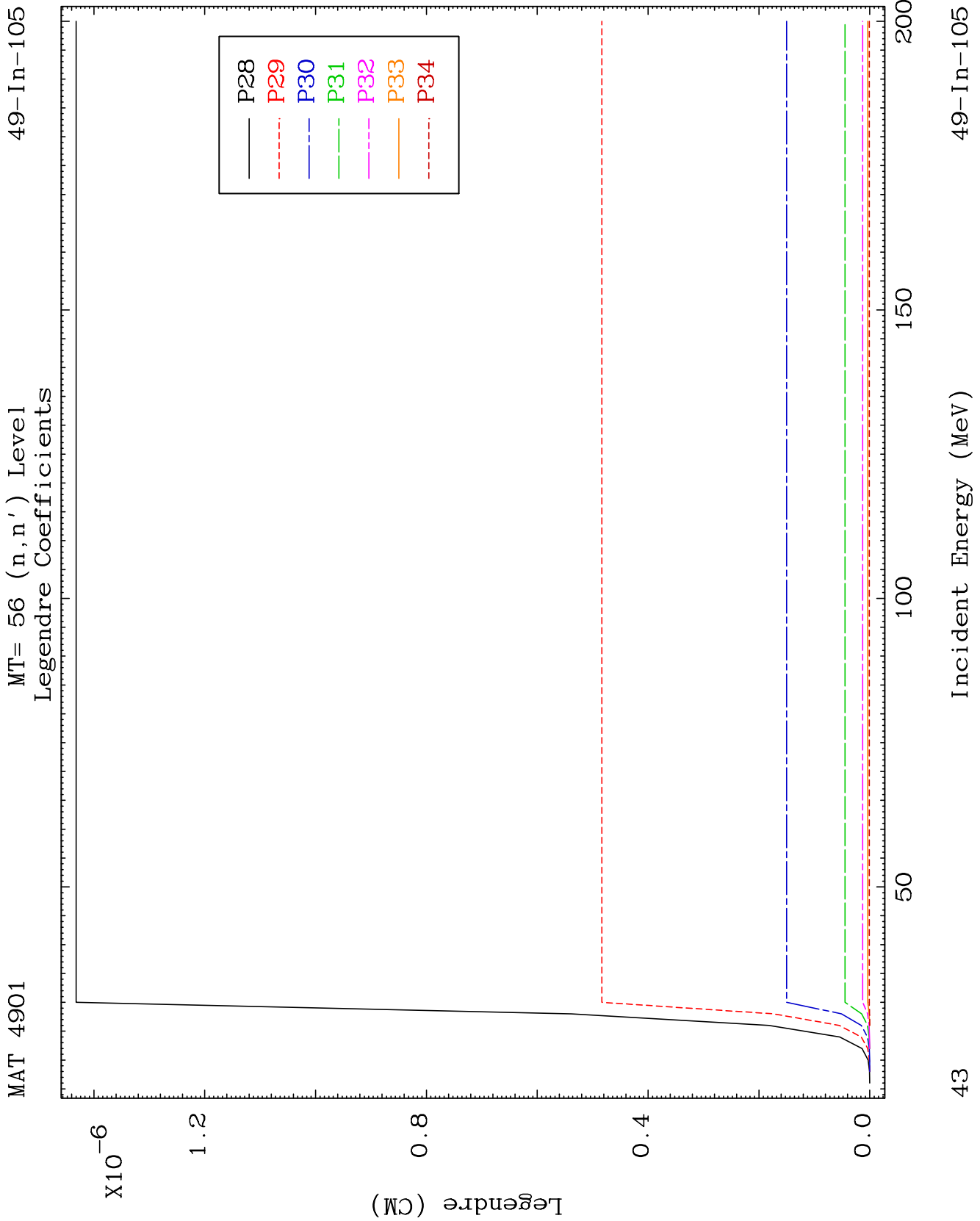
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Legendre Coefficients

49-In-105





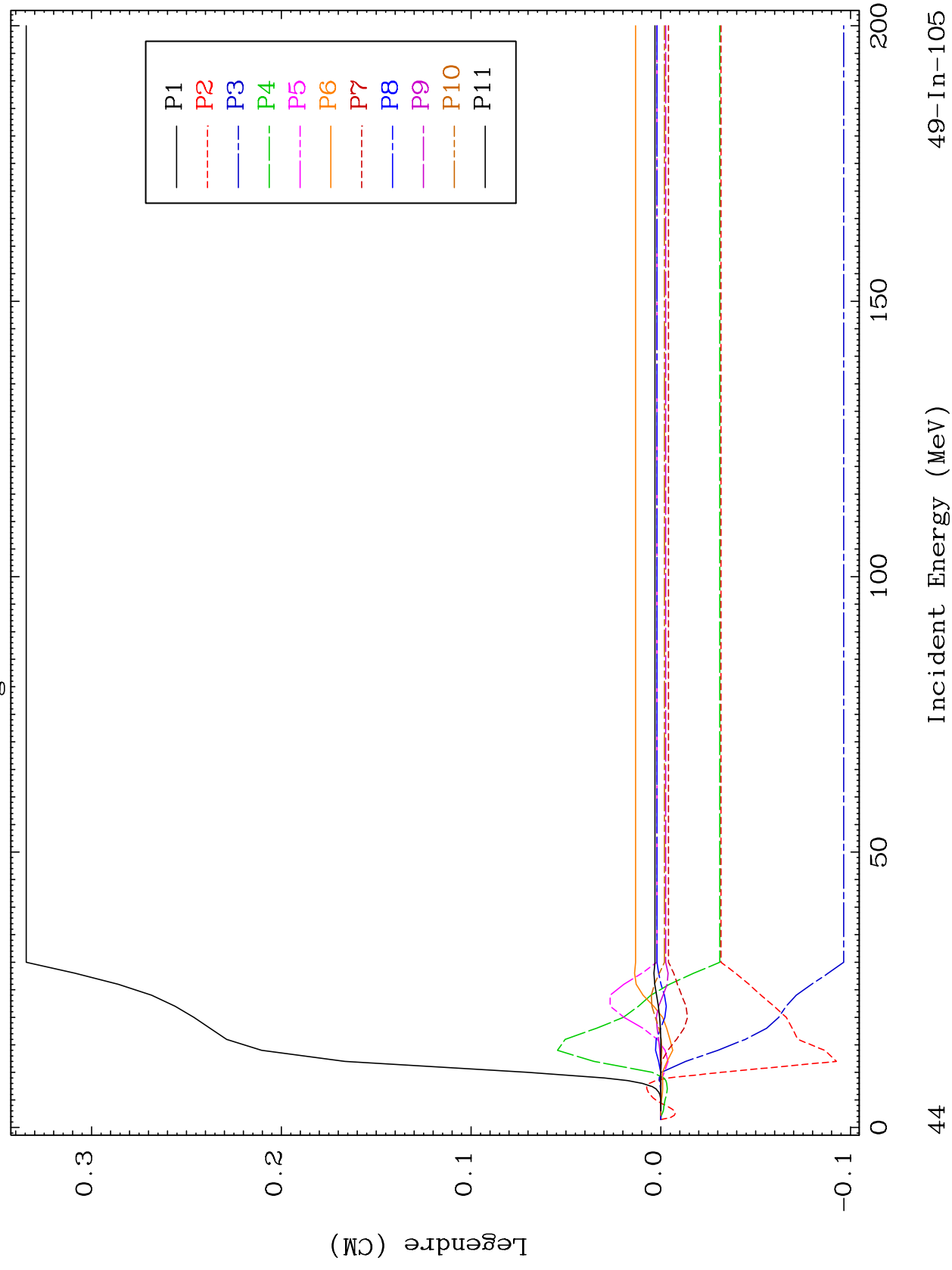


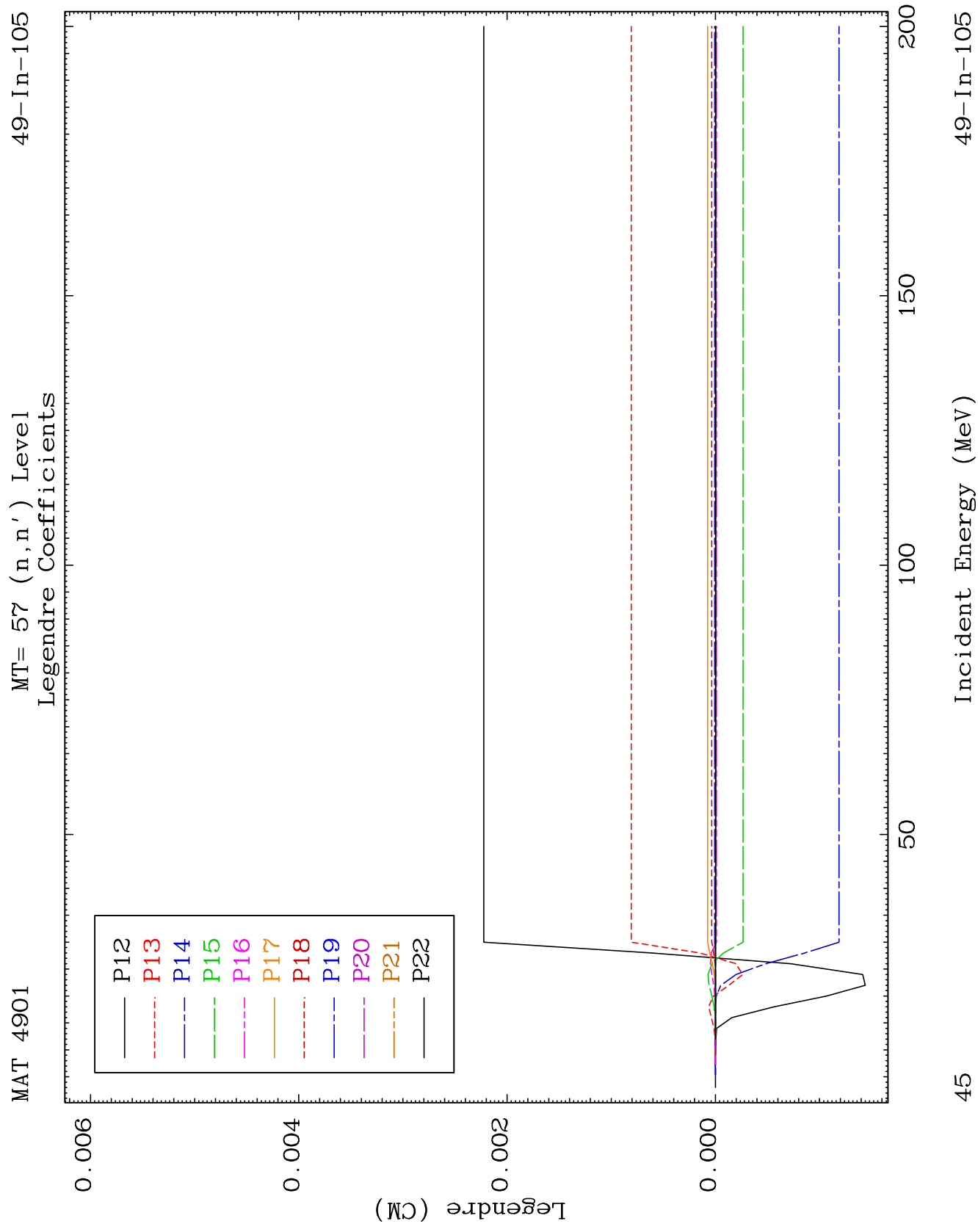


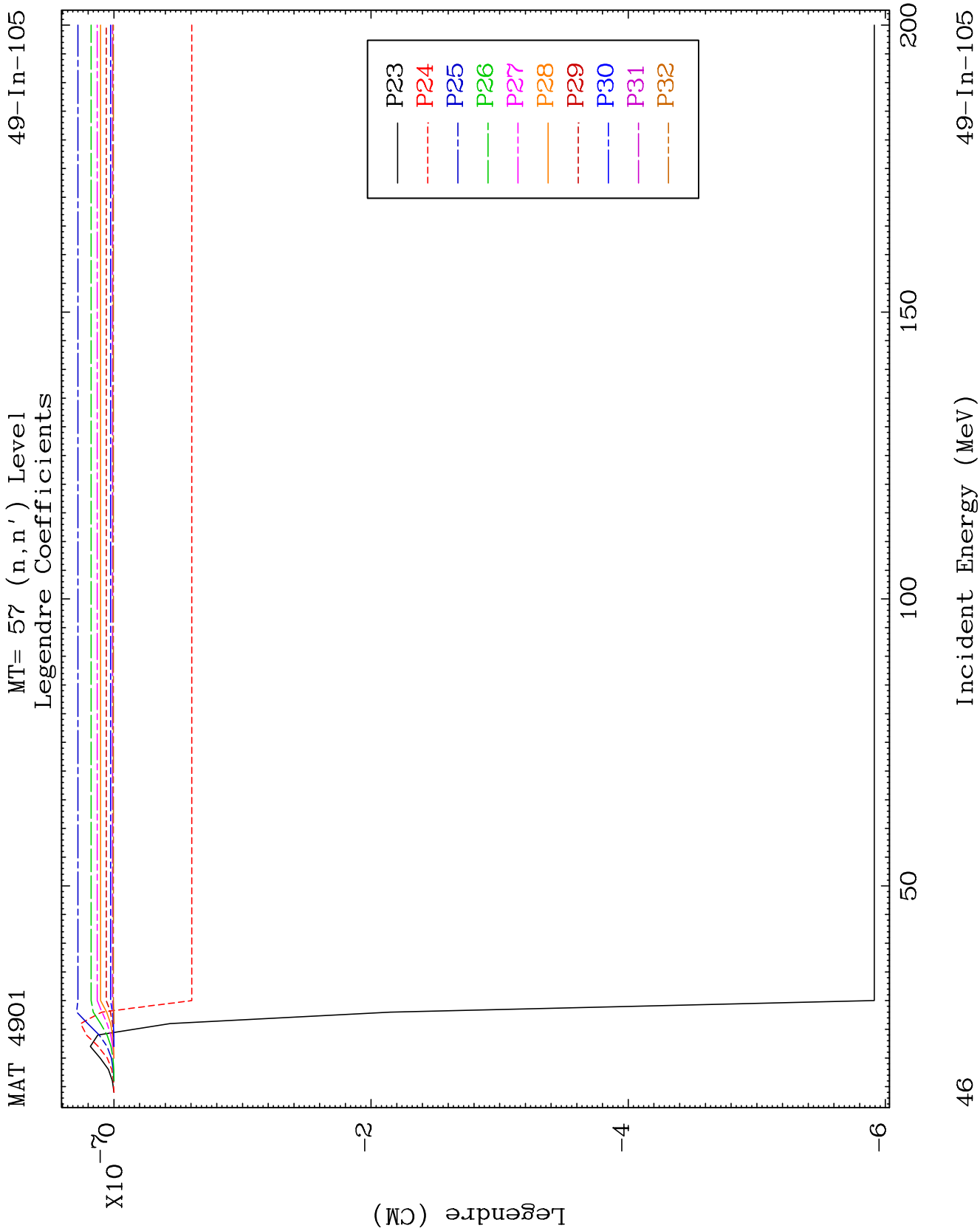
MAT 4901

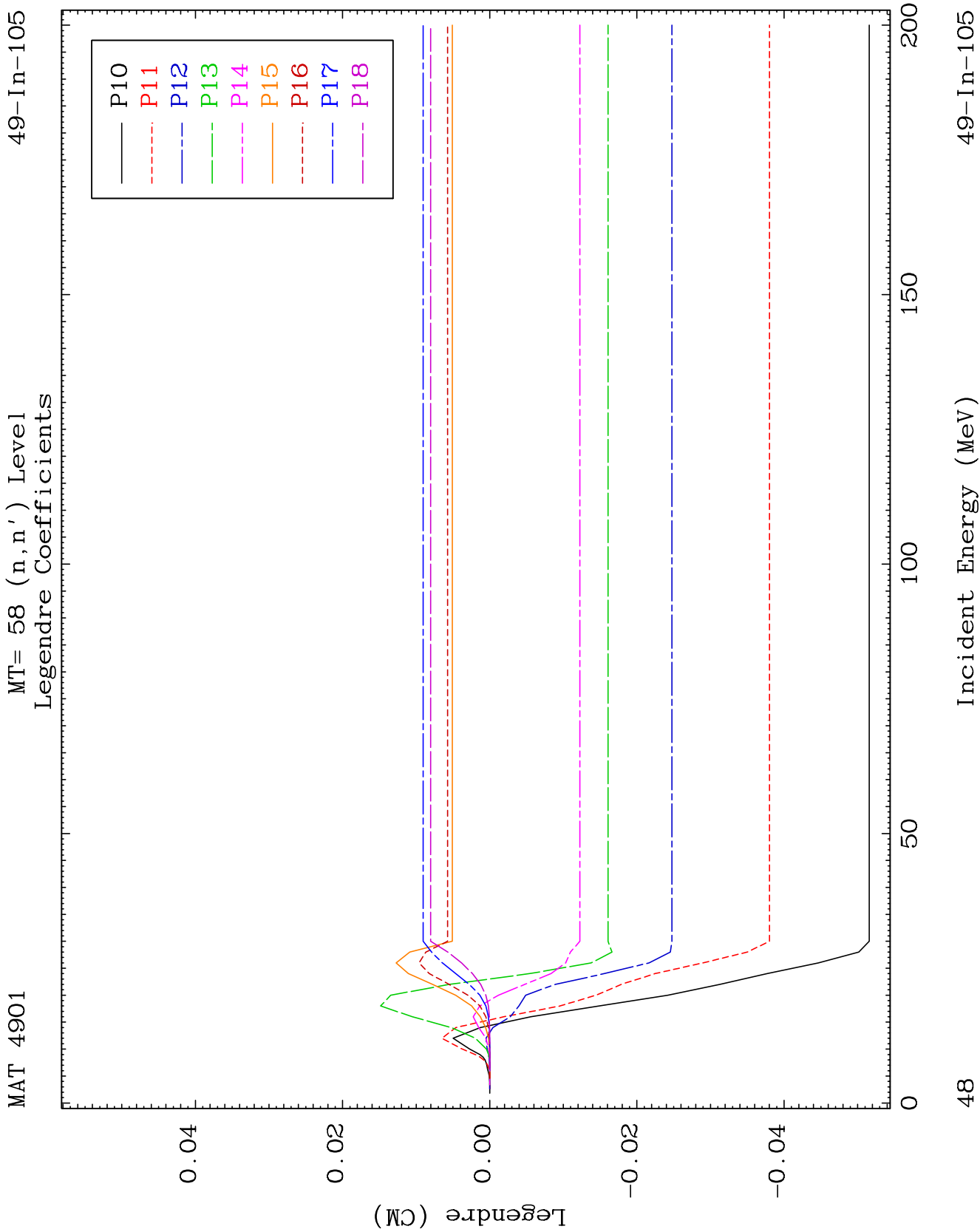
MT= 57 (n, n') Level
Legendre Coefficients

49-In-105





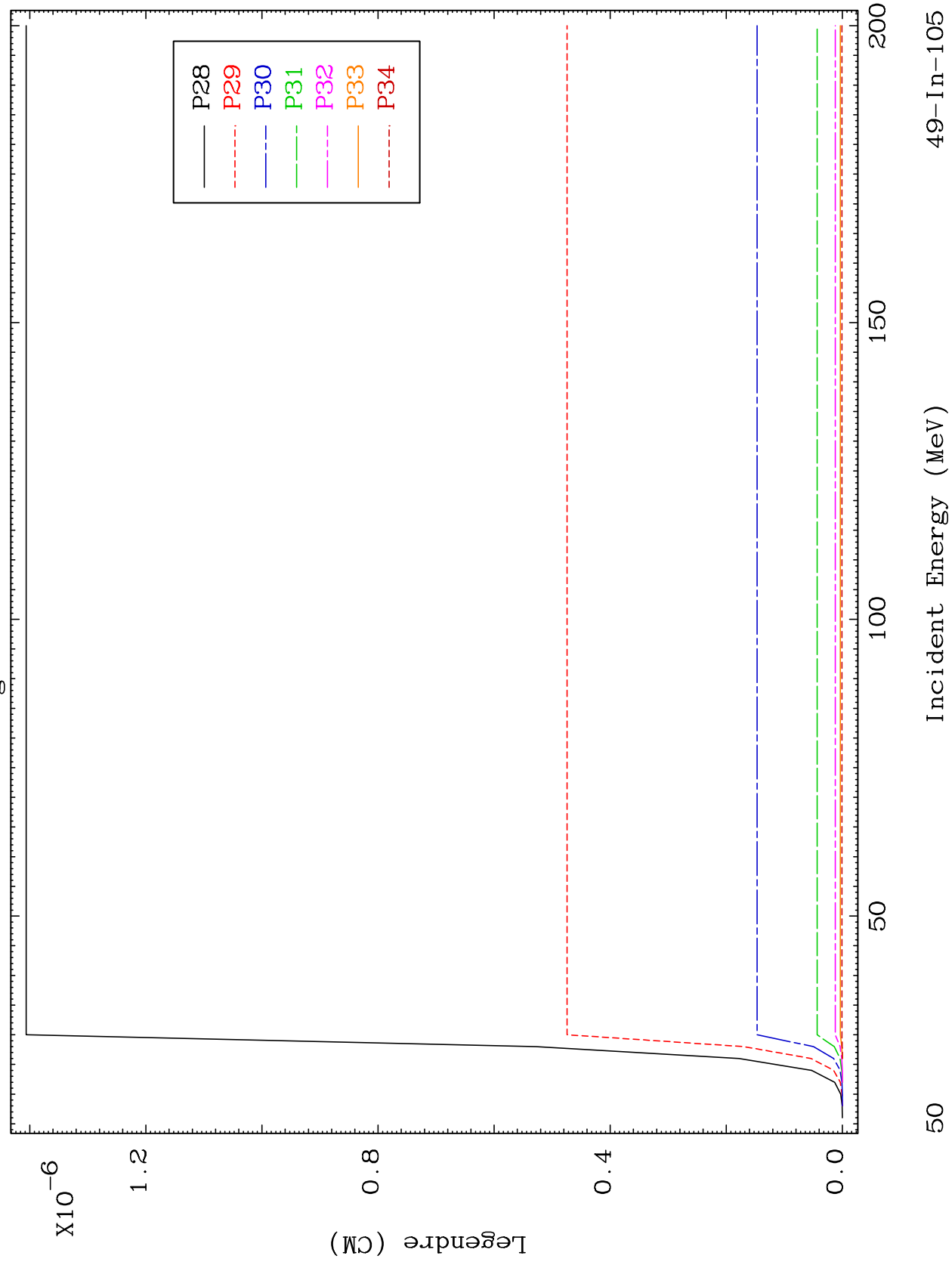


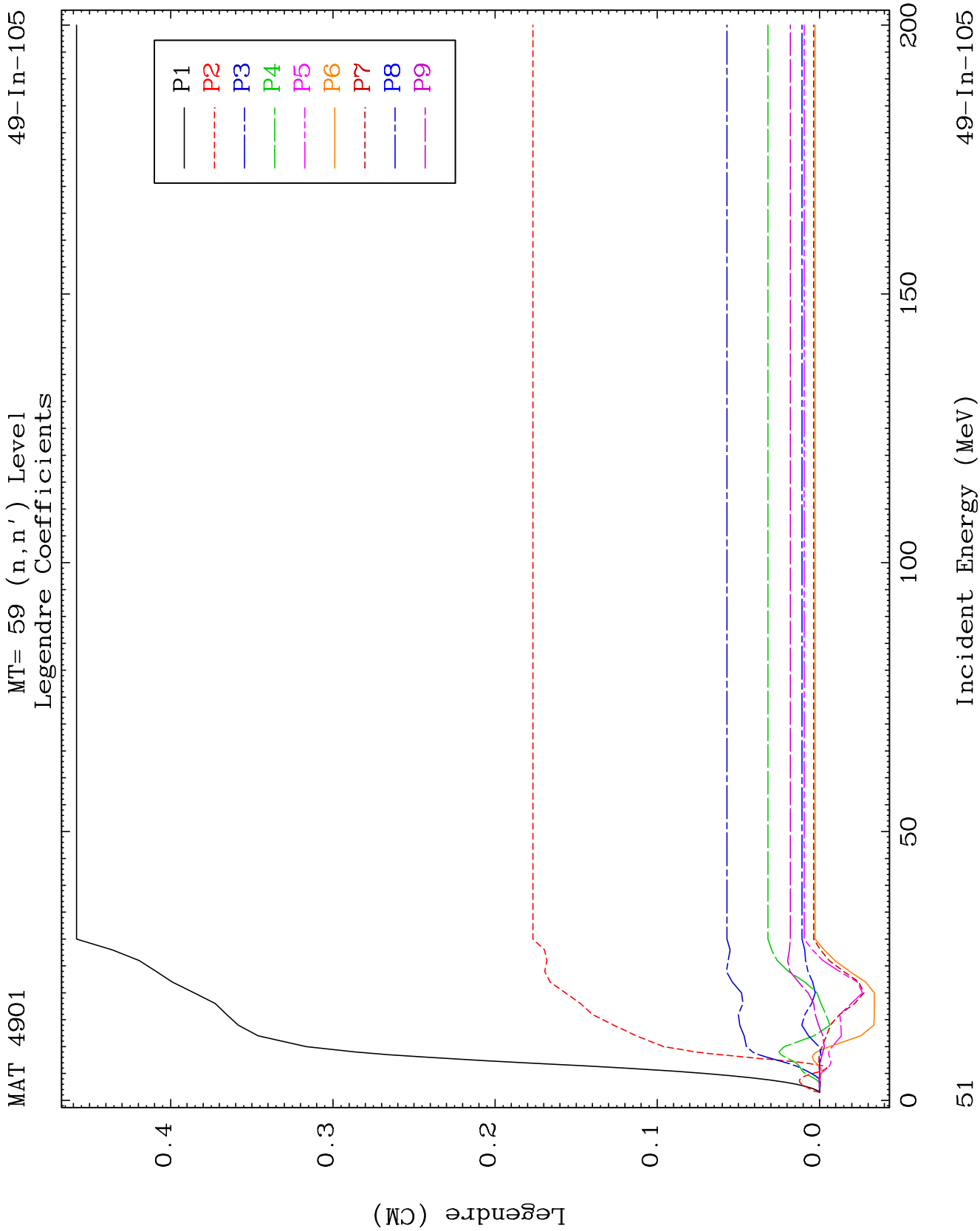


MAT 4901

MT= 58 (n, n') Level
Legendre Coefficients

49-In-105

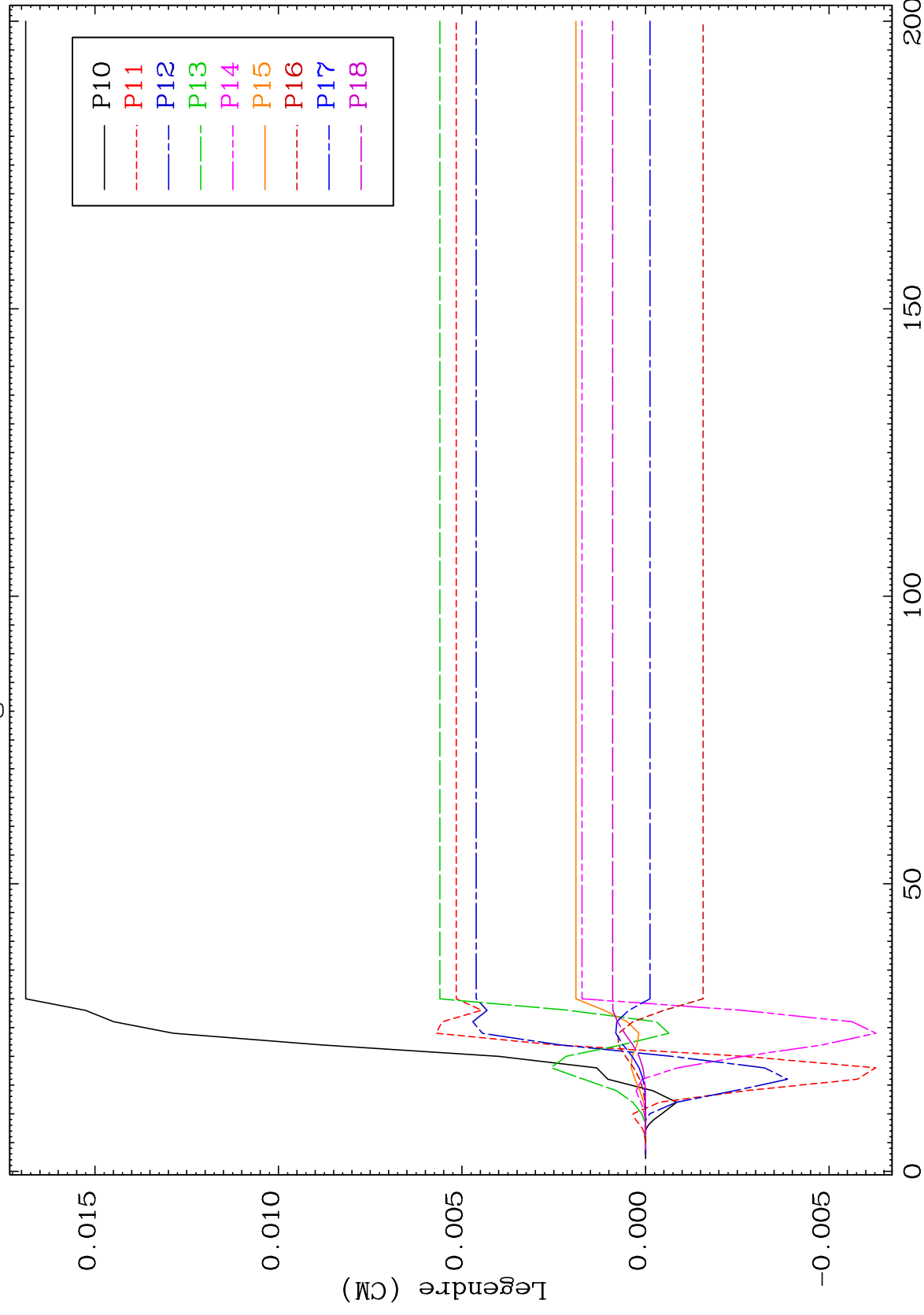




MAT 4901

MT= 59 (n, n') Level
Legendre Coefficients

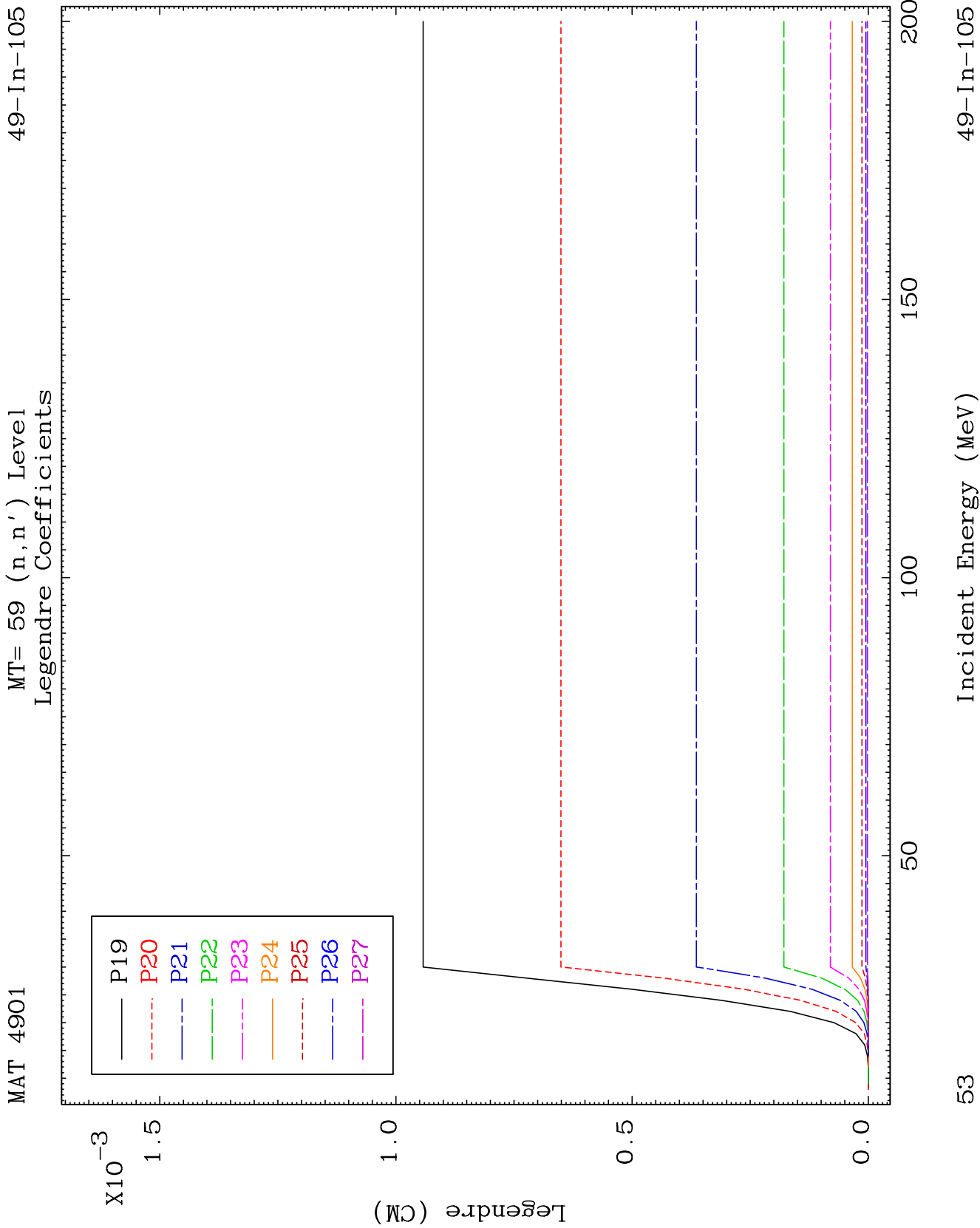
49-In-105

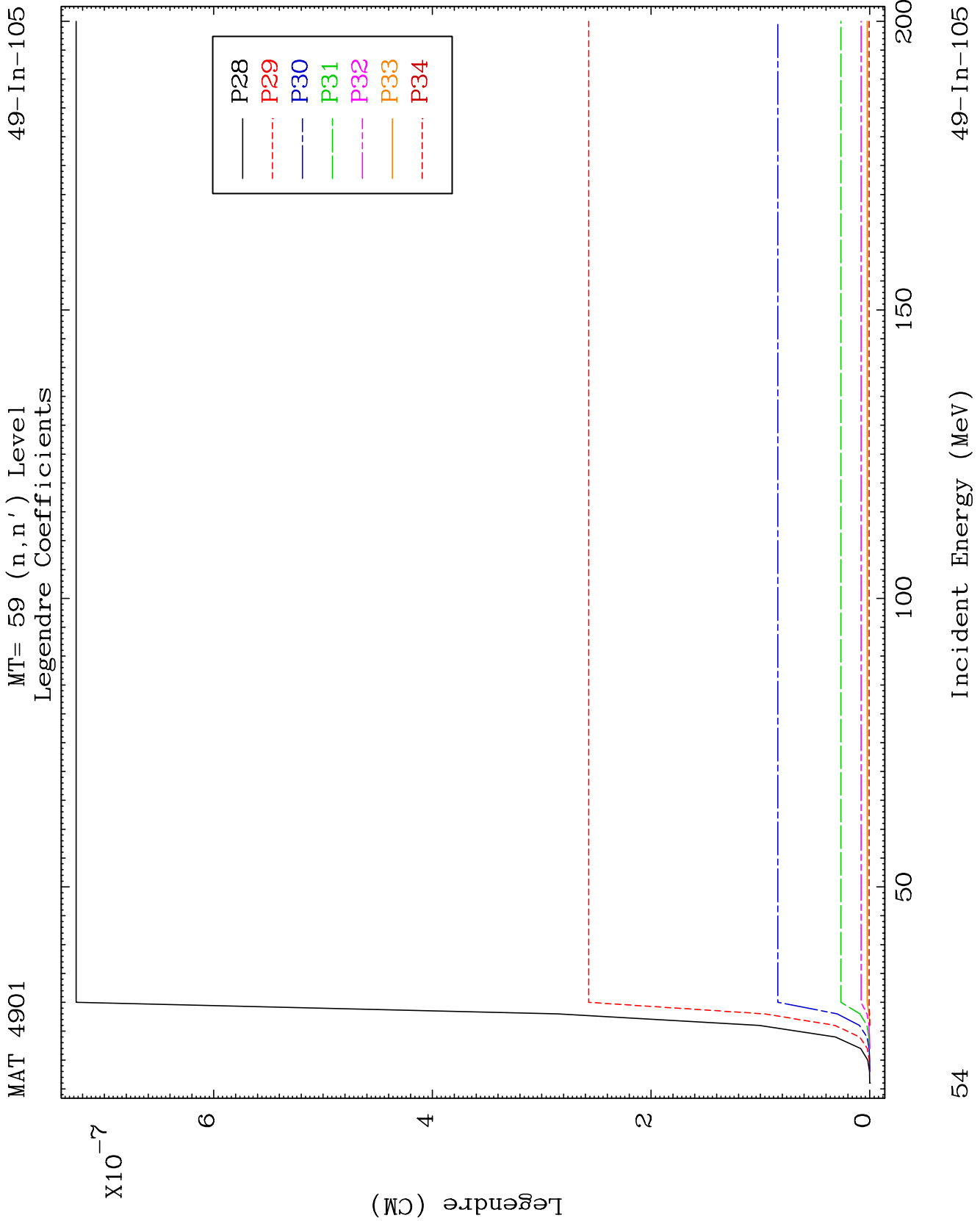


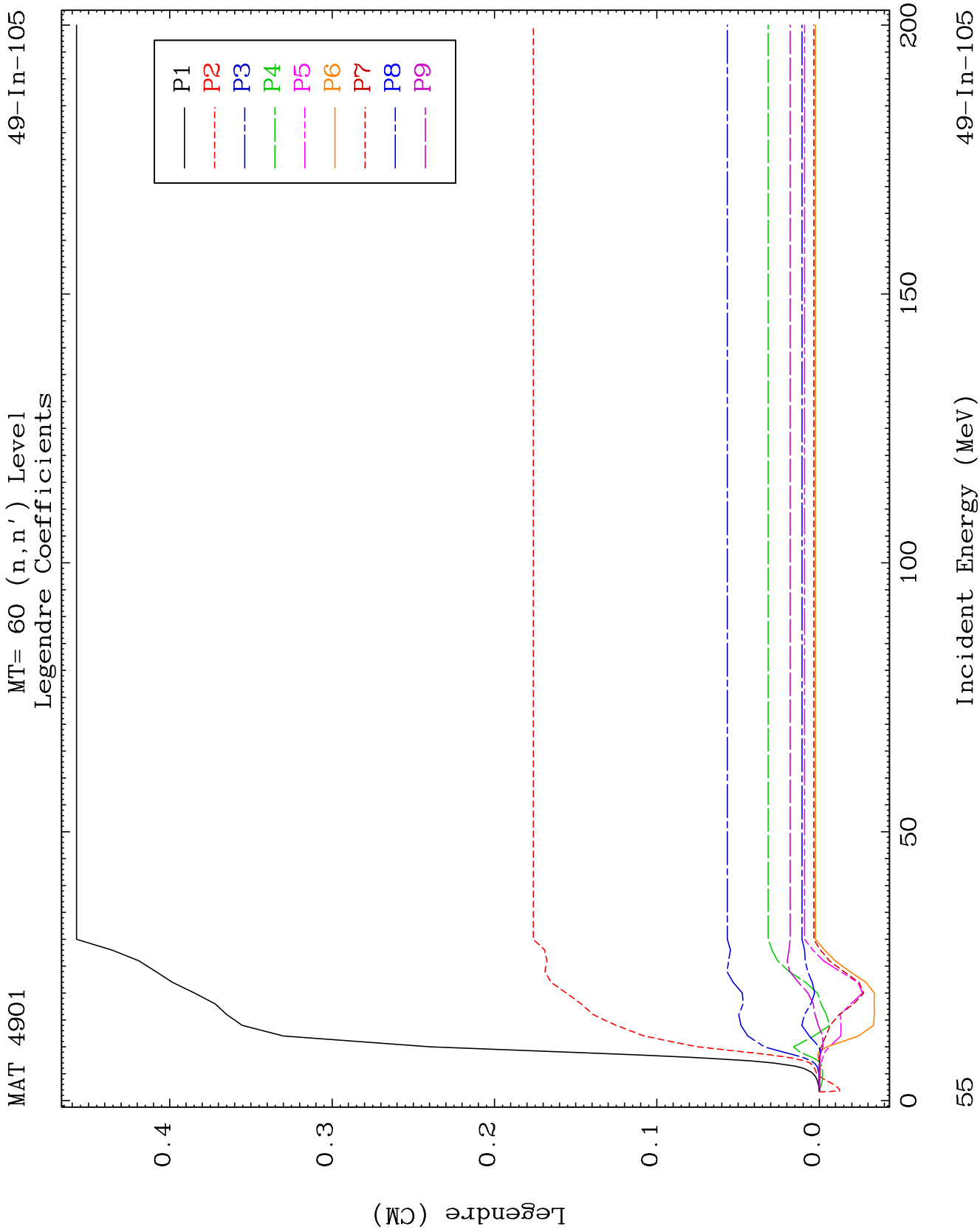
52

Incident Energy (MeV)

49-In-105



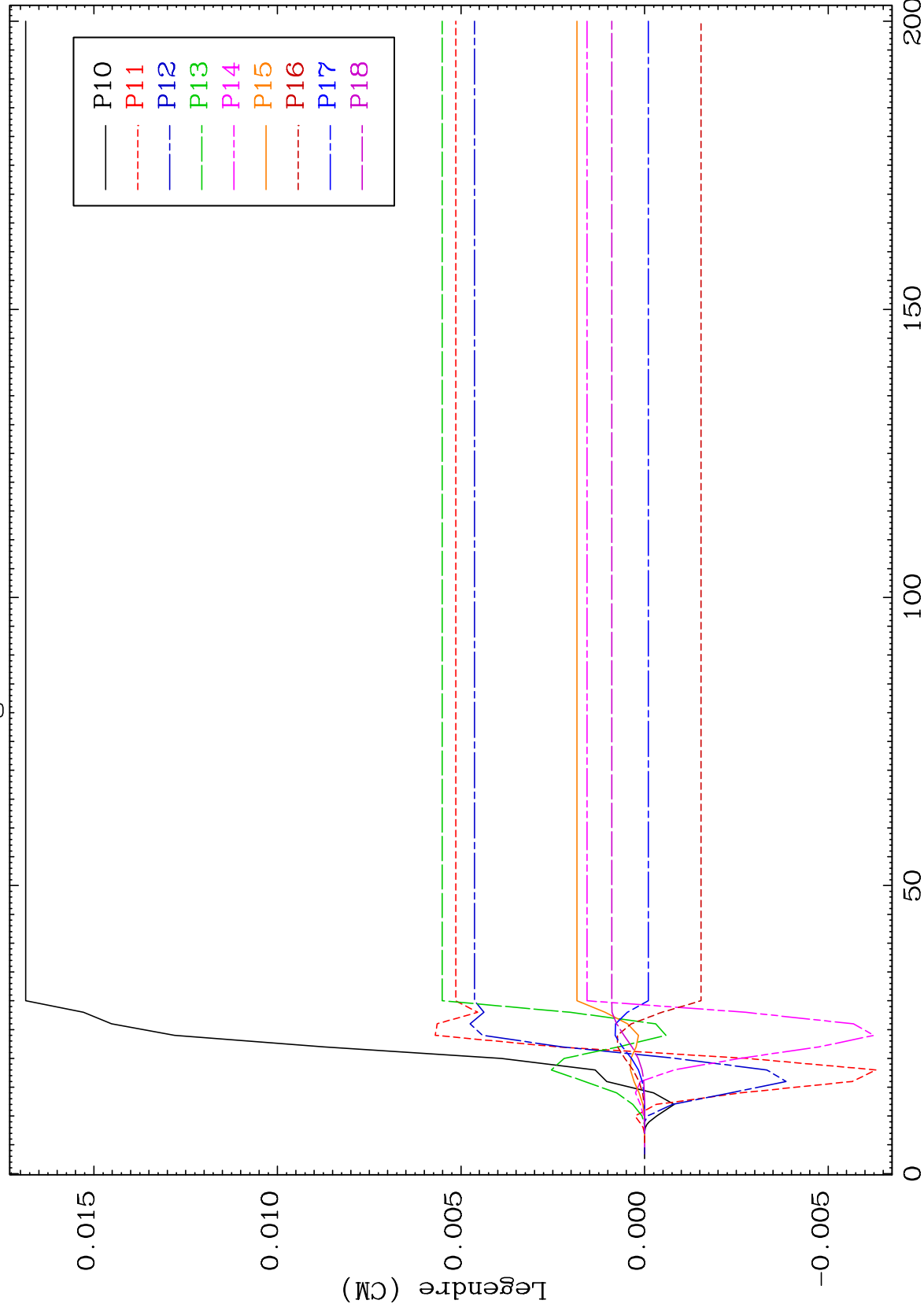




MAT 4901

MT= 60 (n, n') Level
Legendre Coefficients

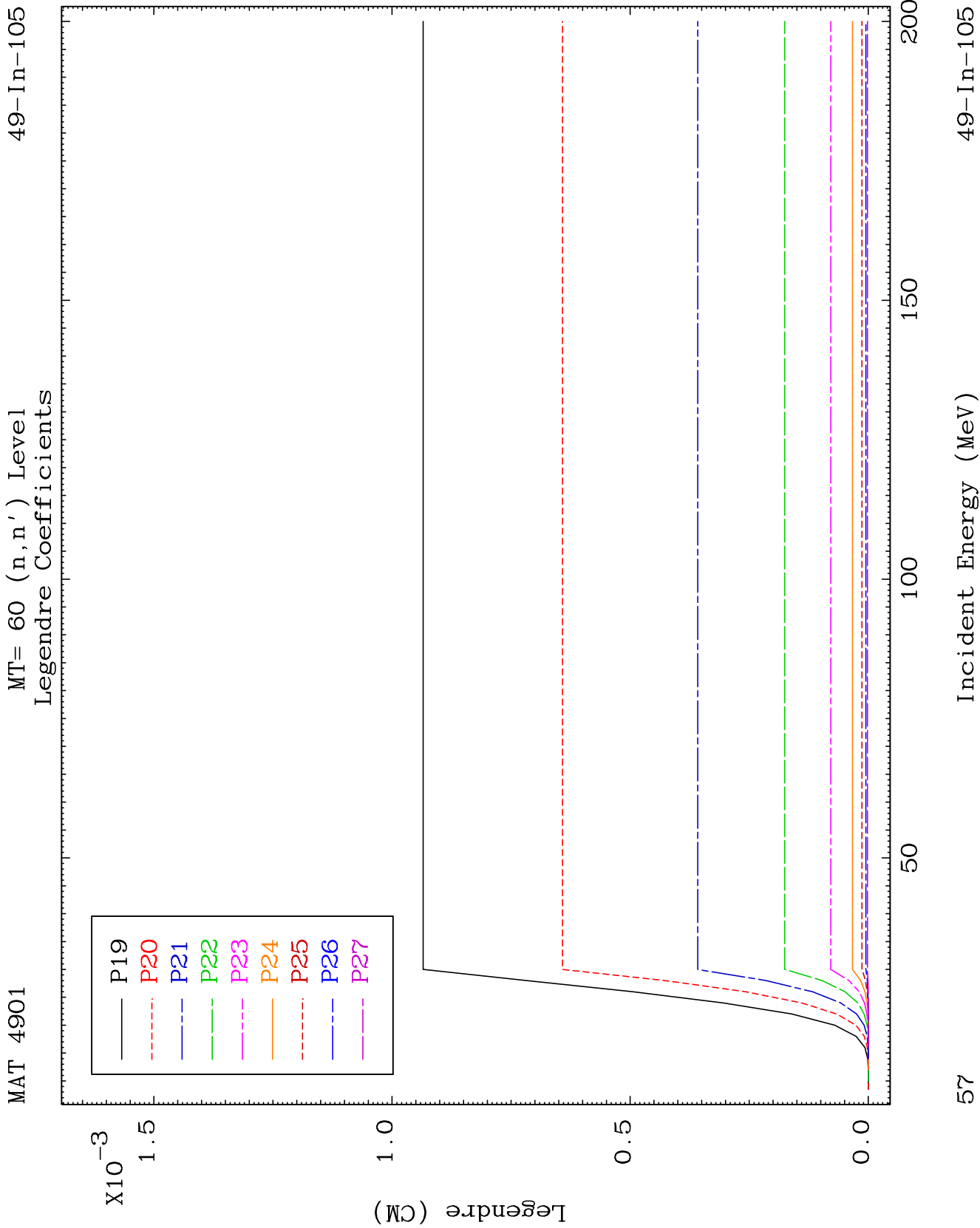
49-In-105

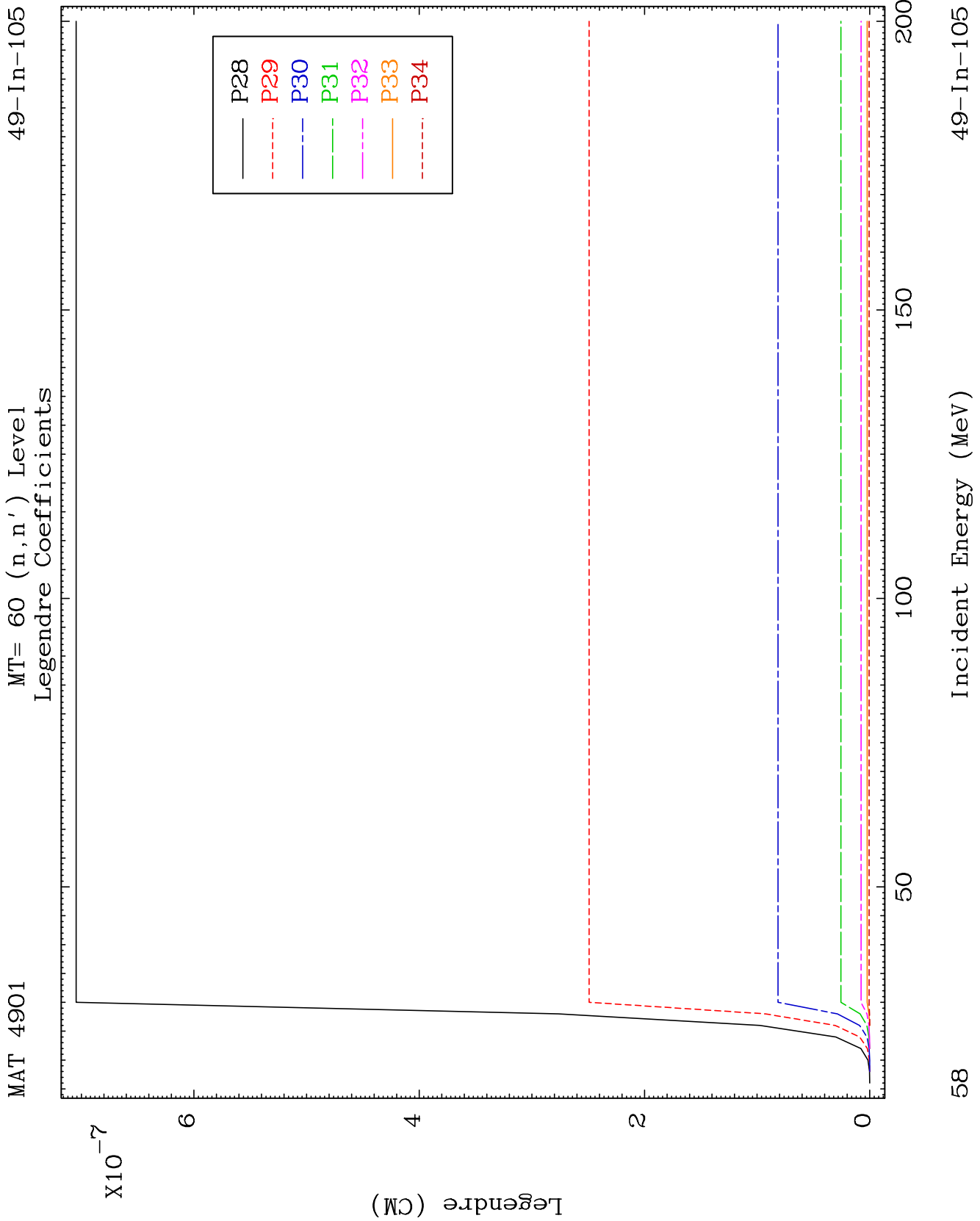


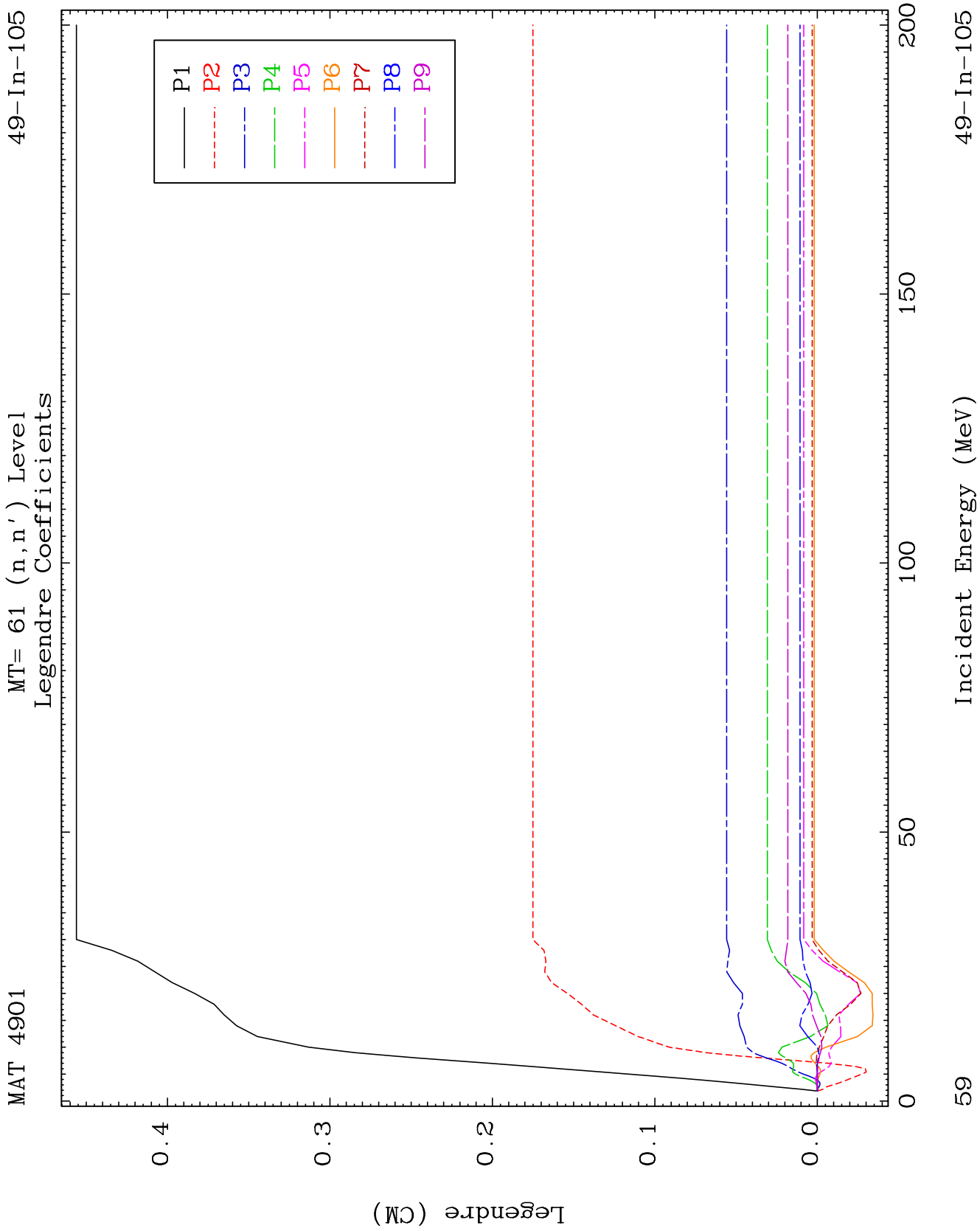
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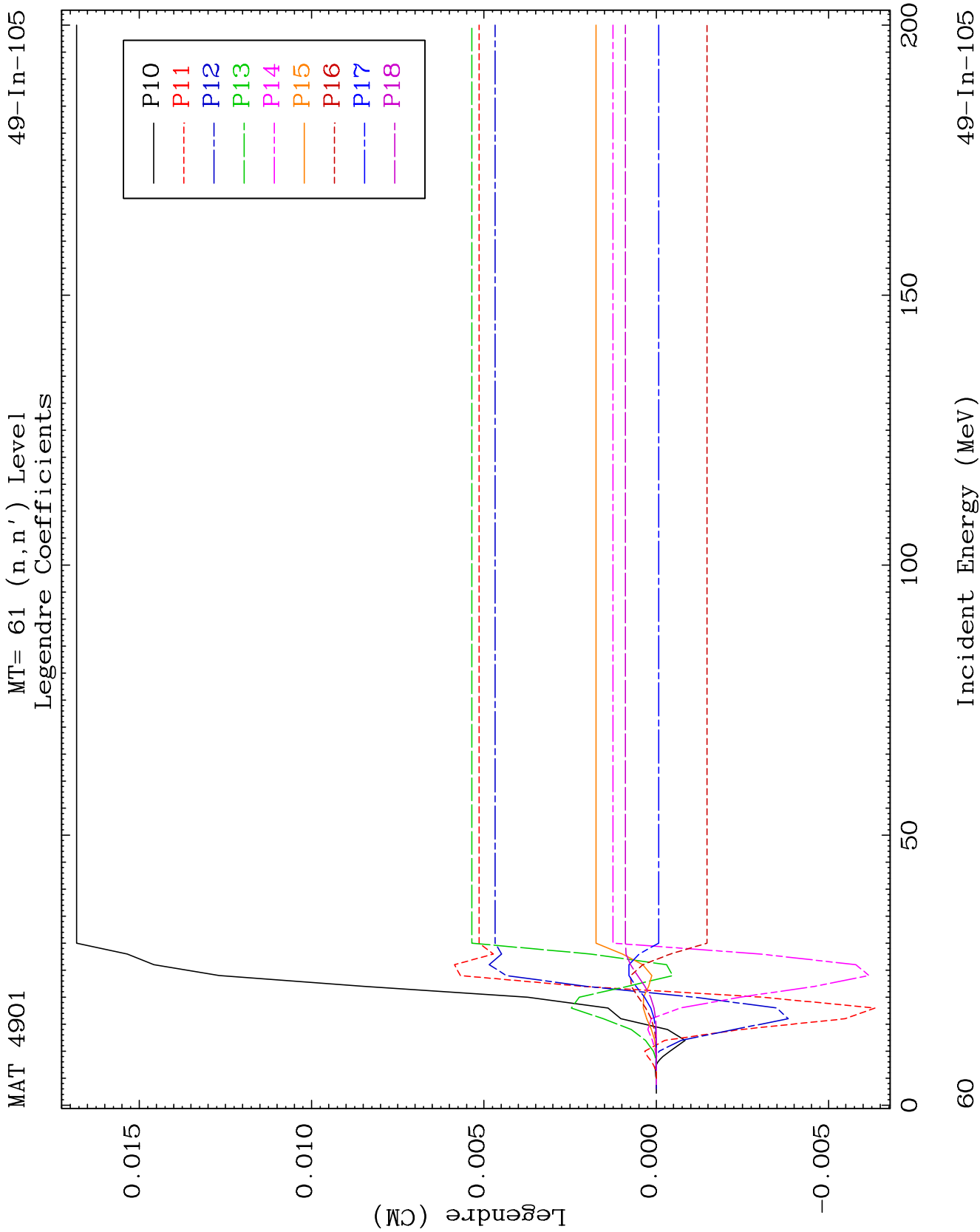
Incident Energy (MeV)

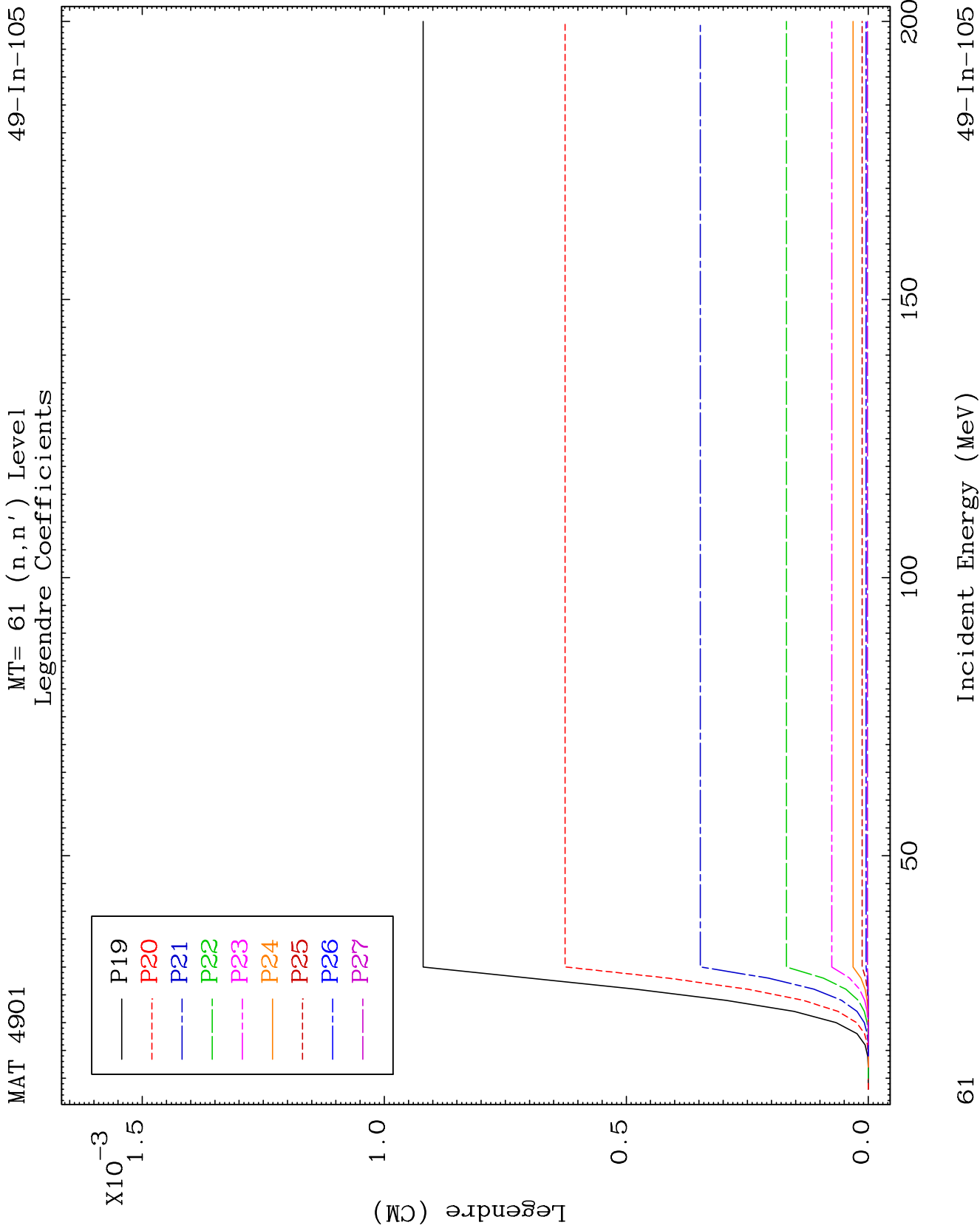
49-In-105

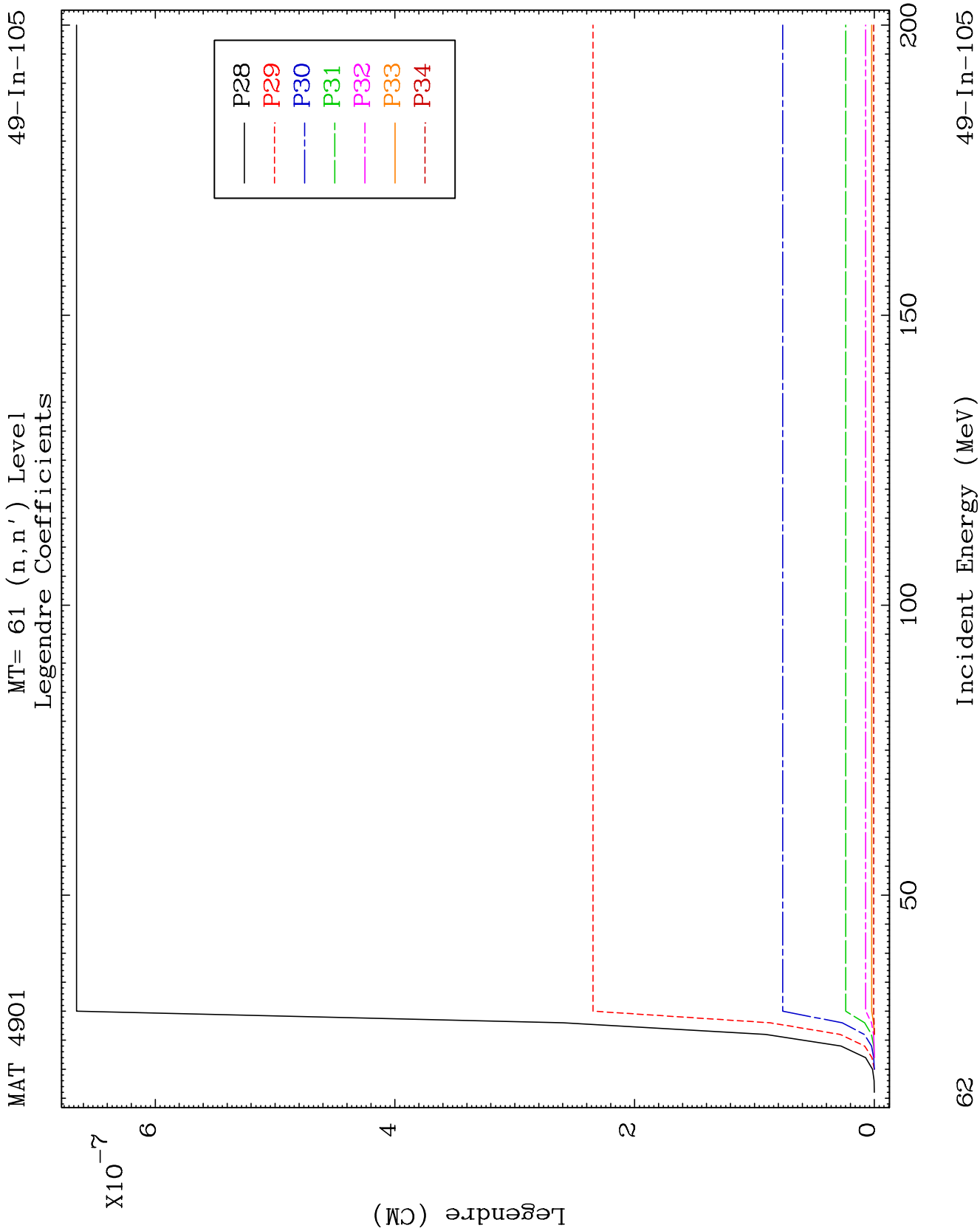


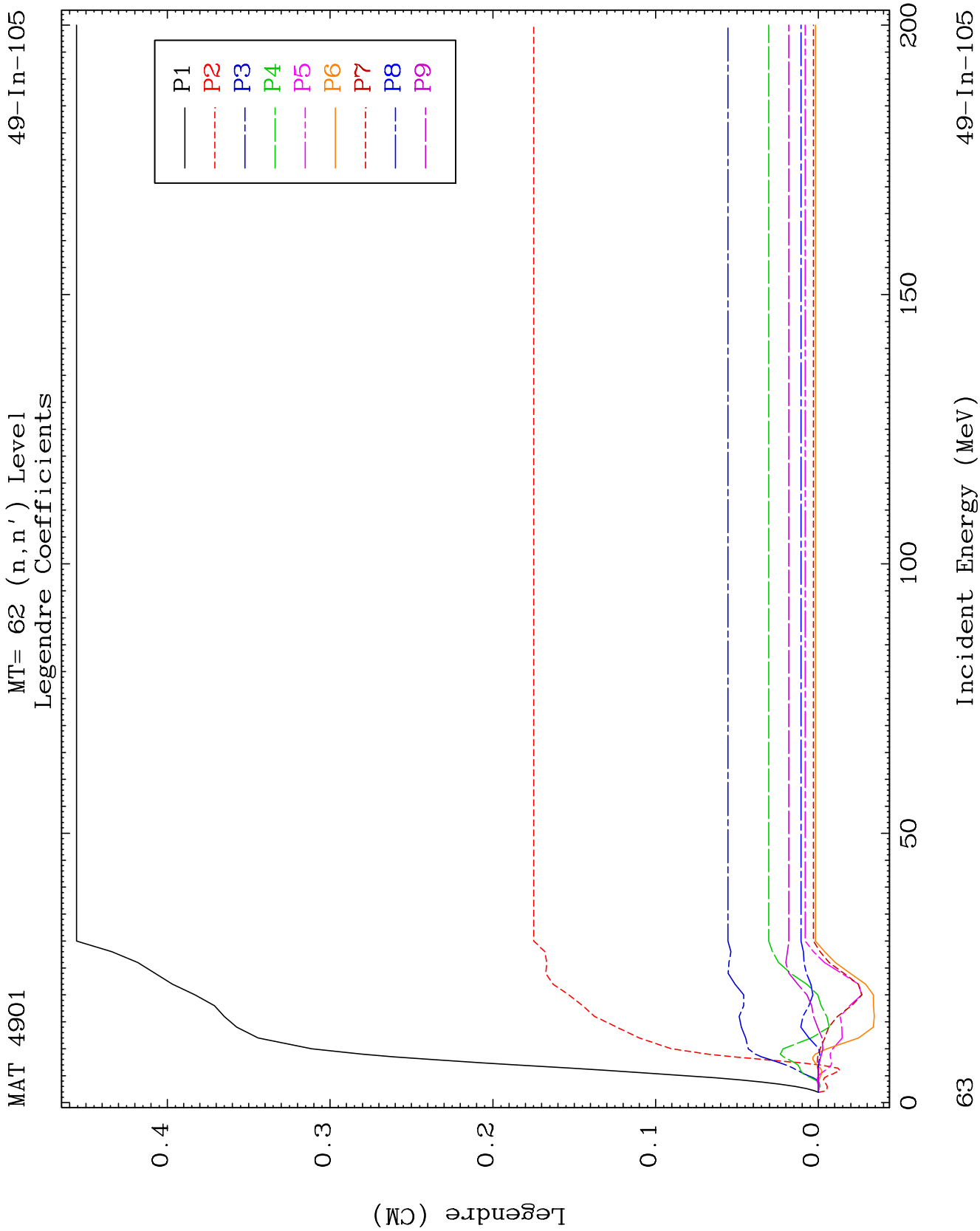


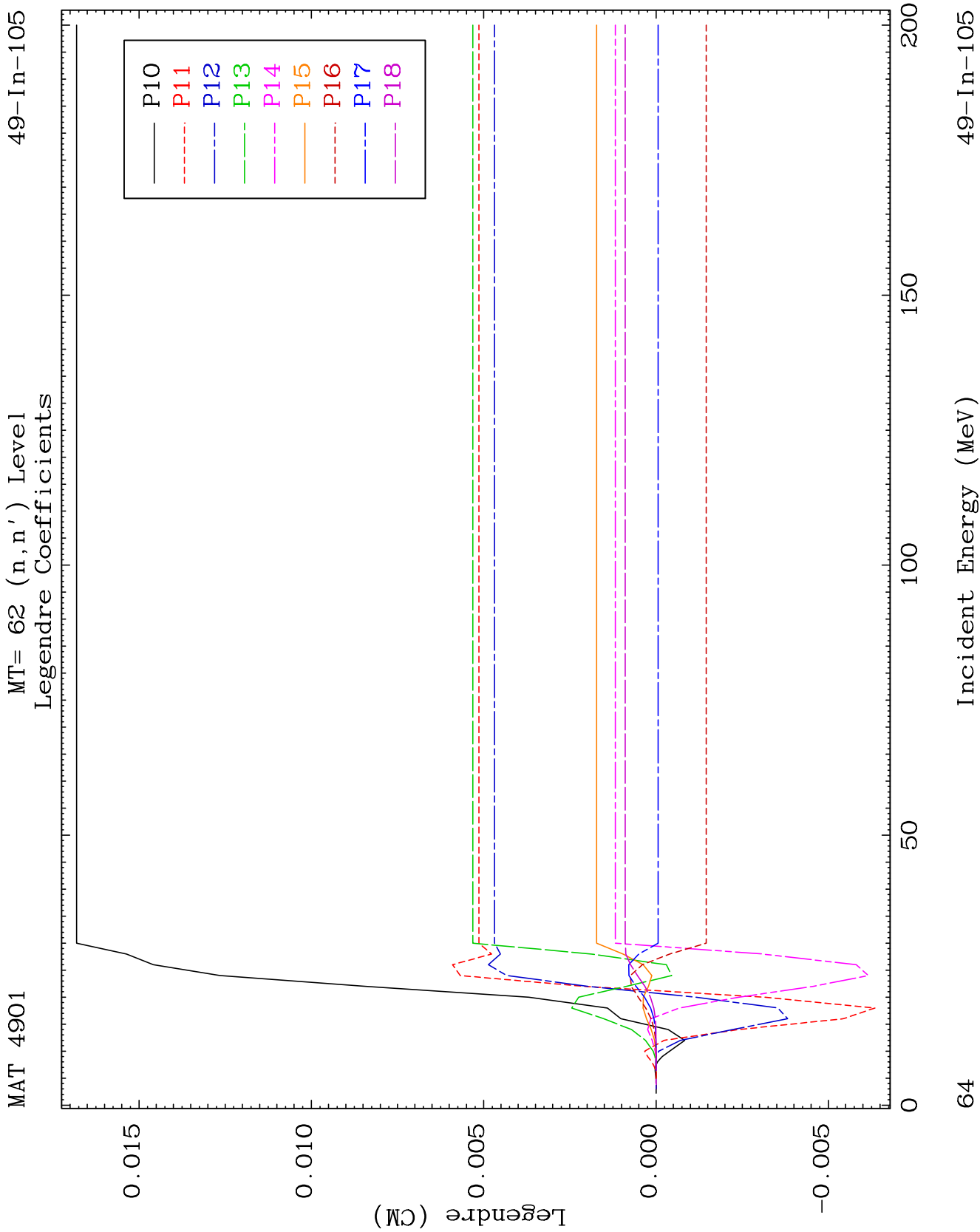


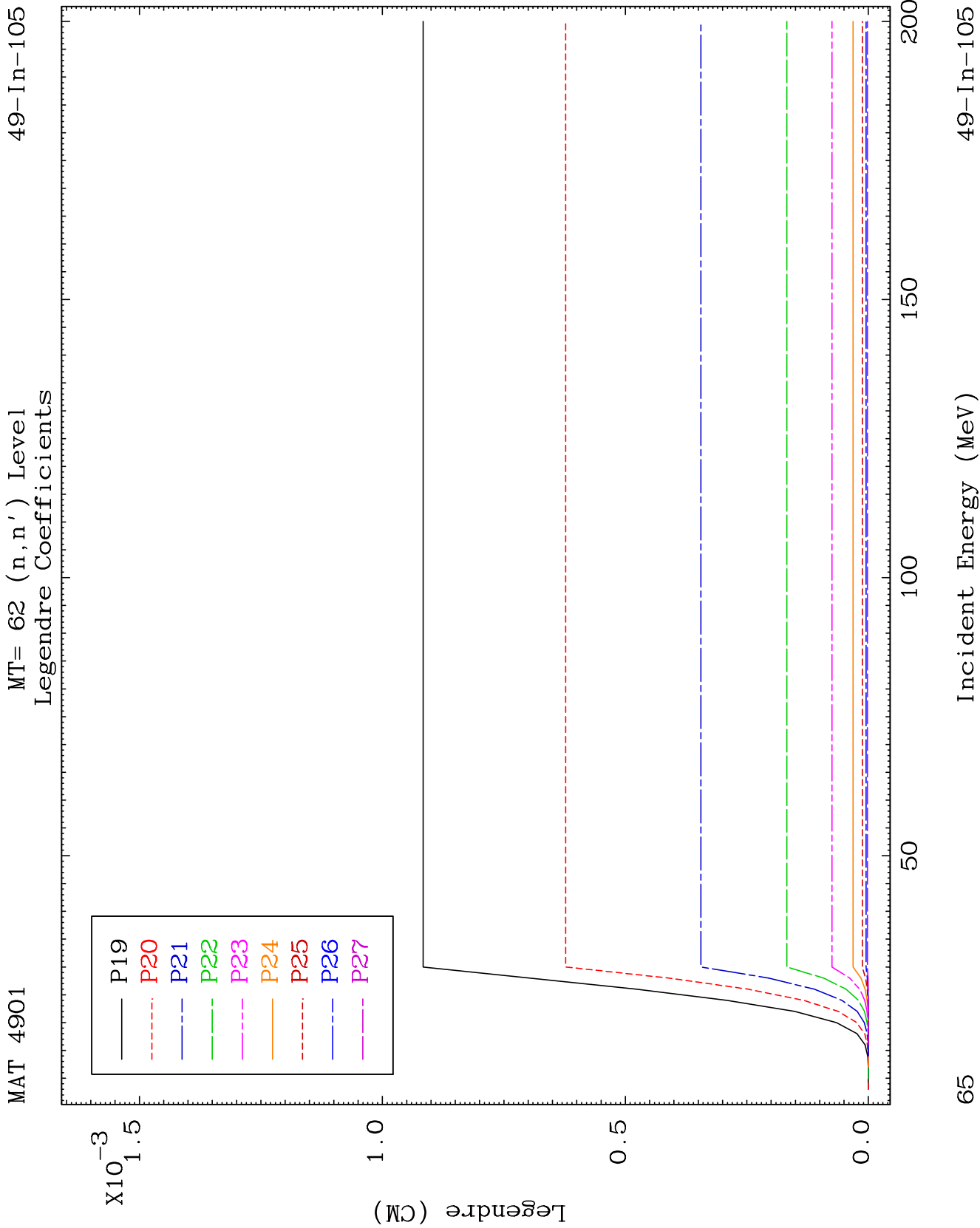


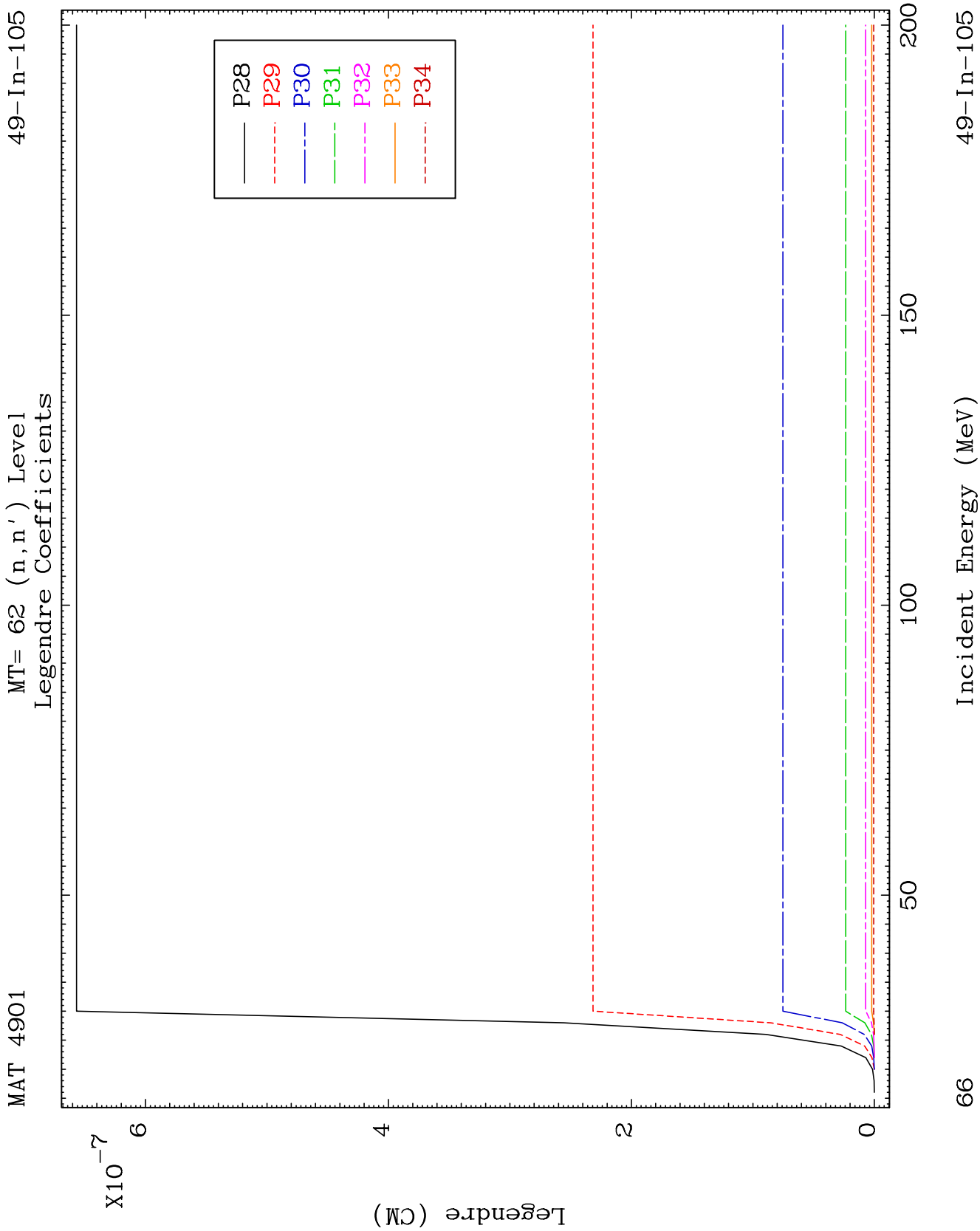


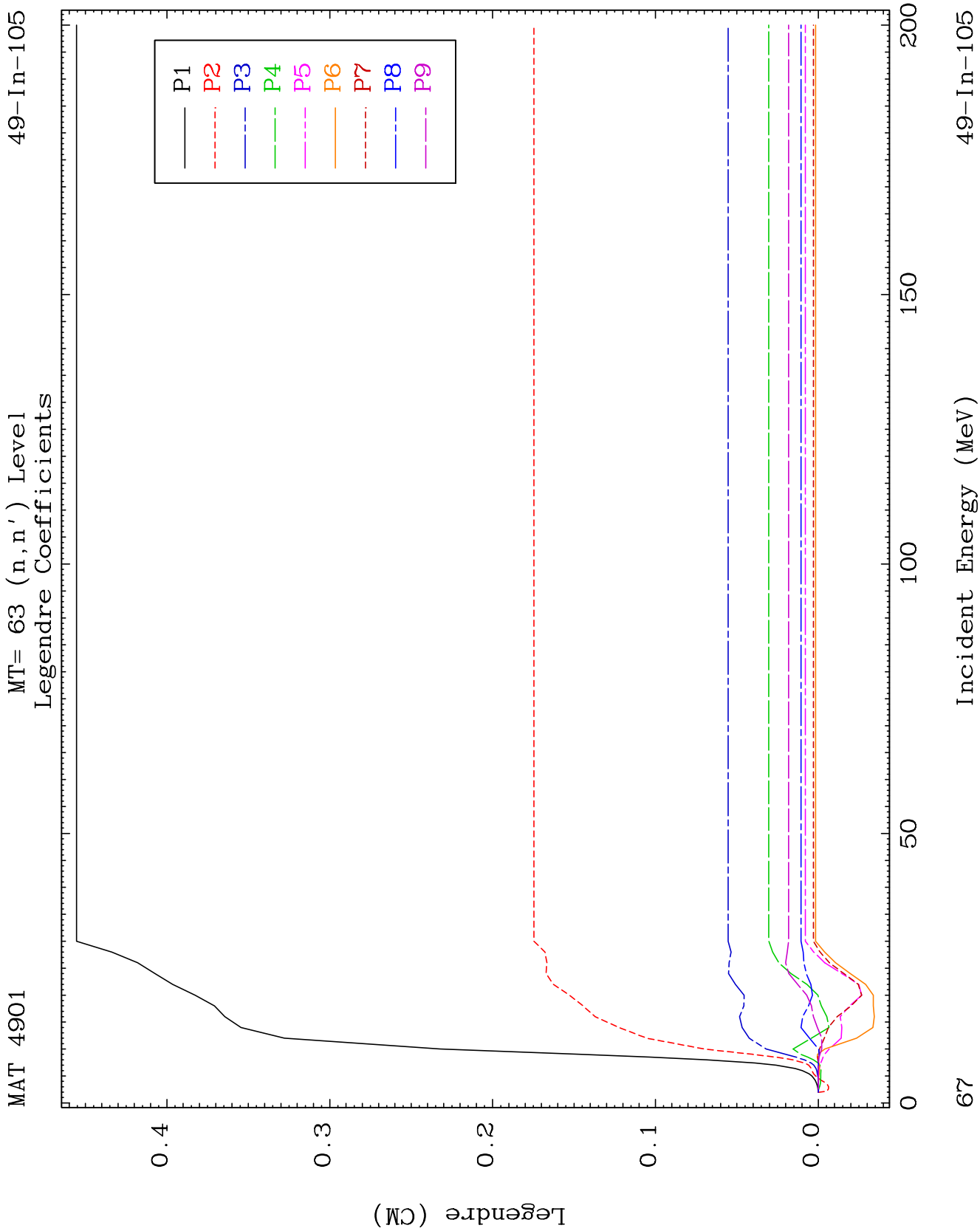








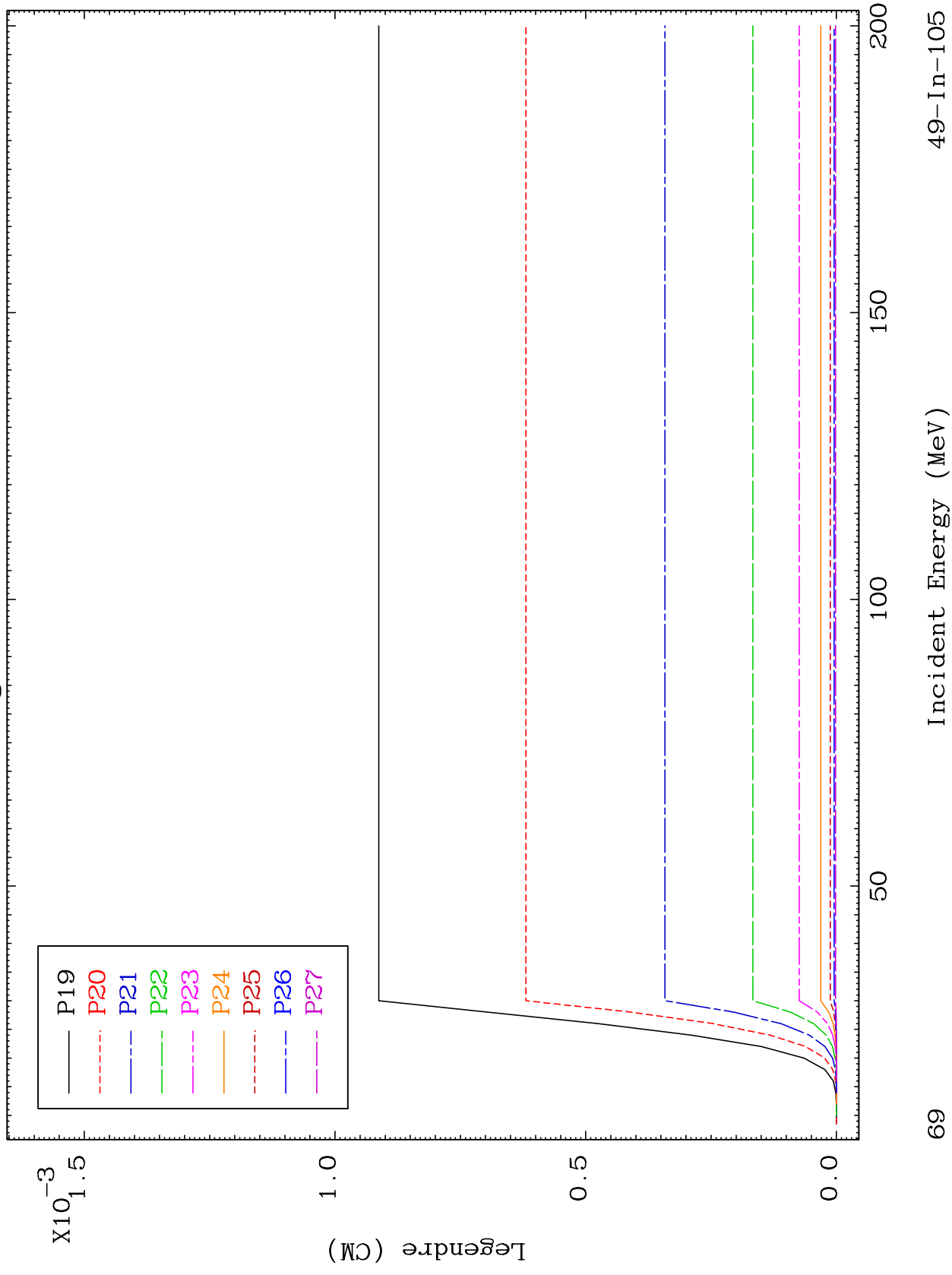


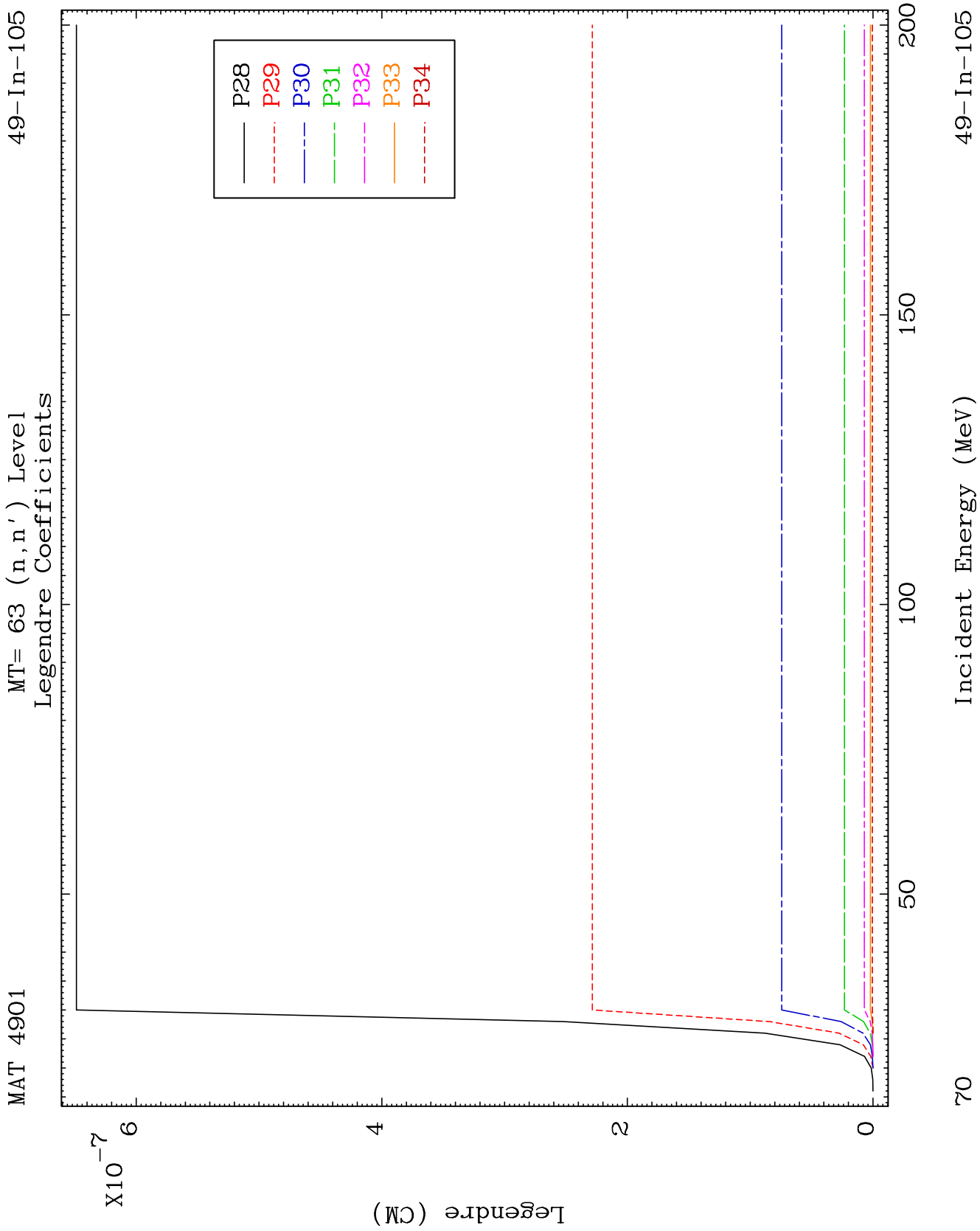


MAT 4901

MT= 63 (n, n') Level
Legendre Coefficients

49-In-105

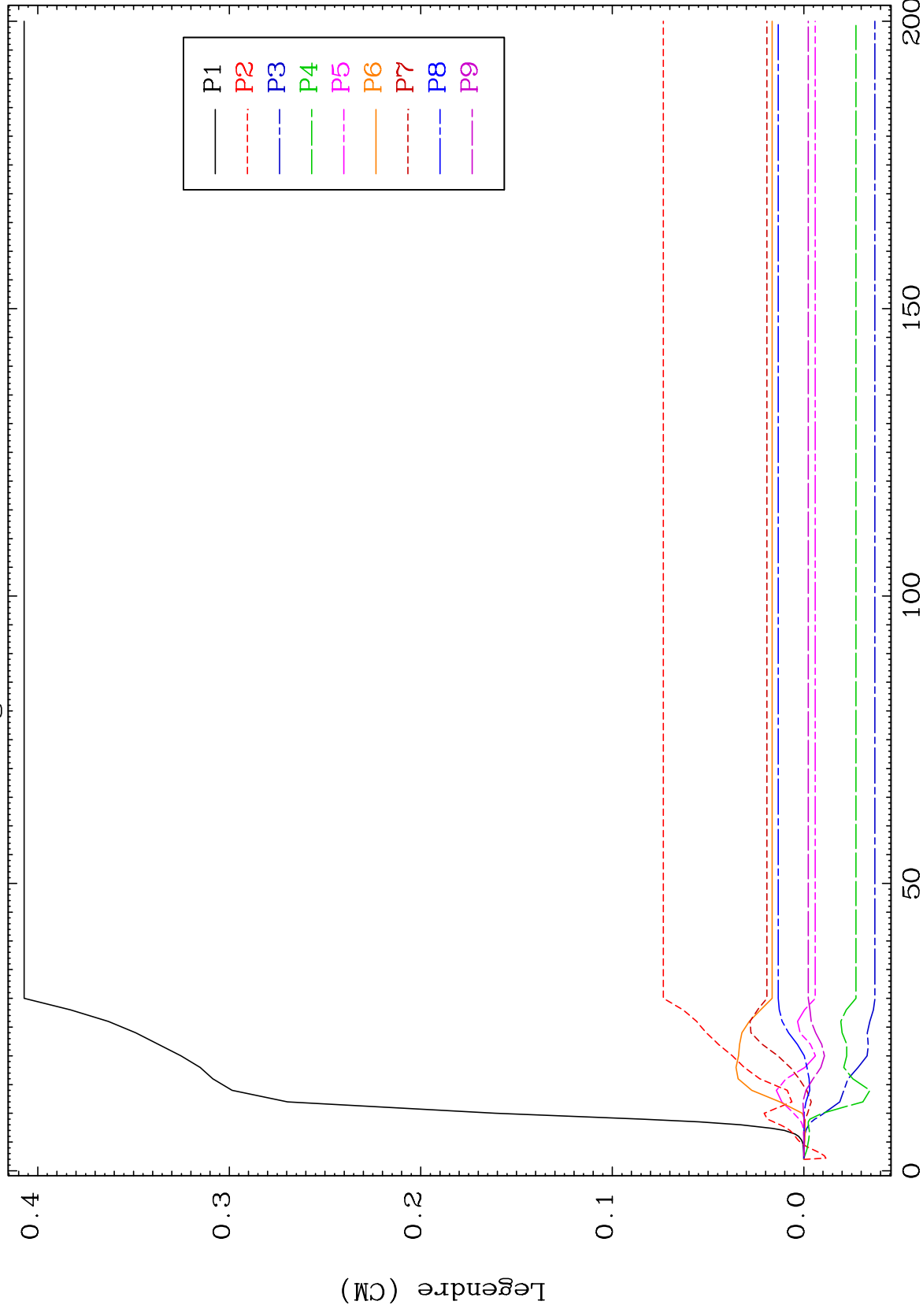




MAT 4901

MT= 64 (n, n') Level
Legendre Coefficients

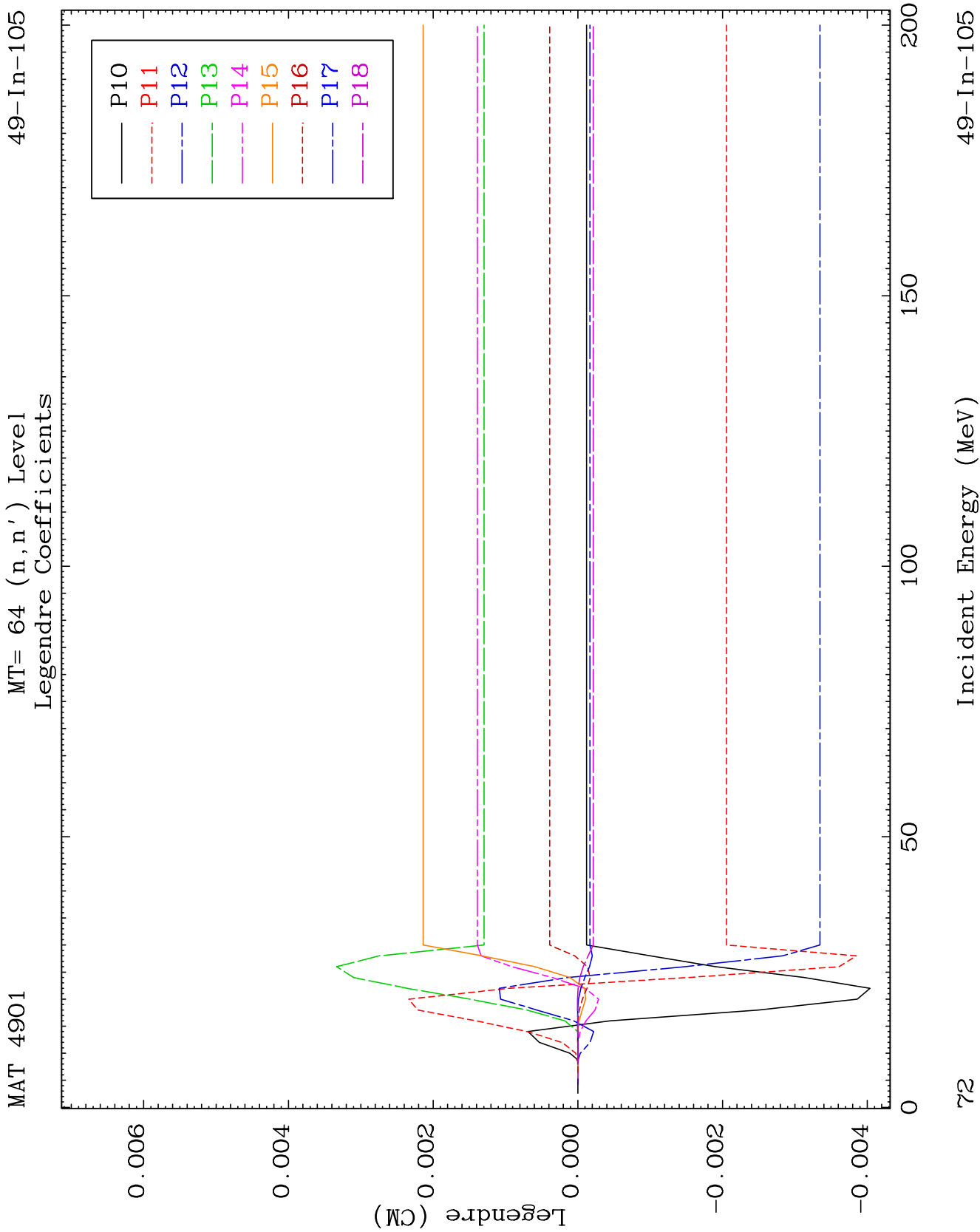
49-In-105

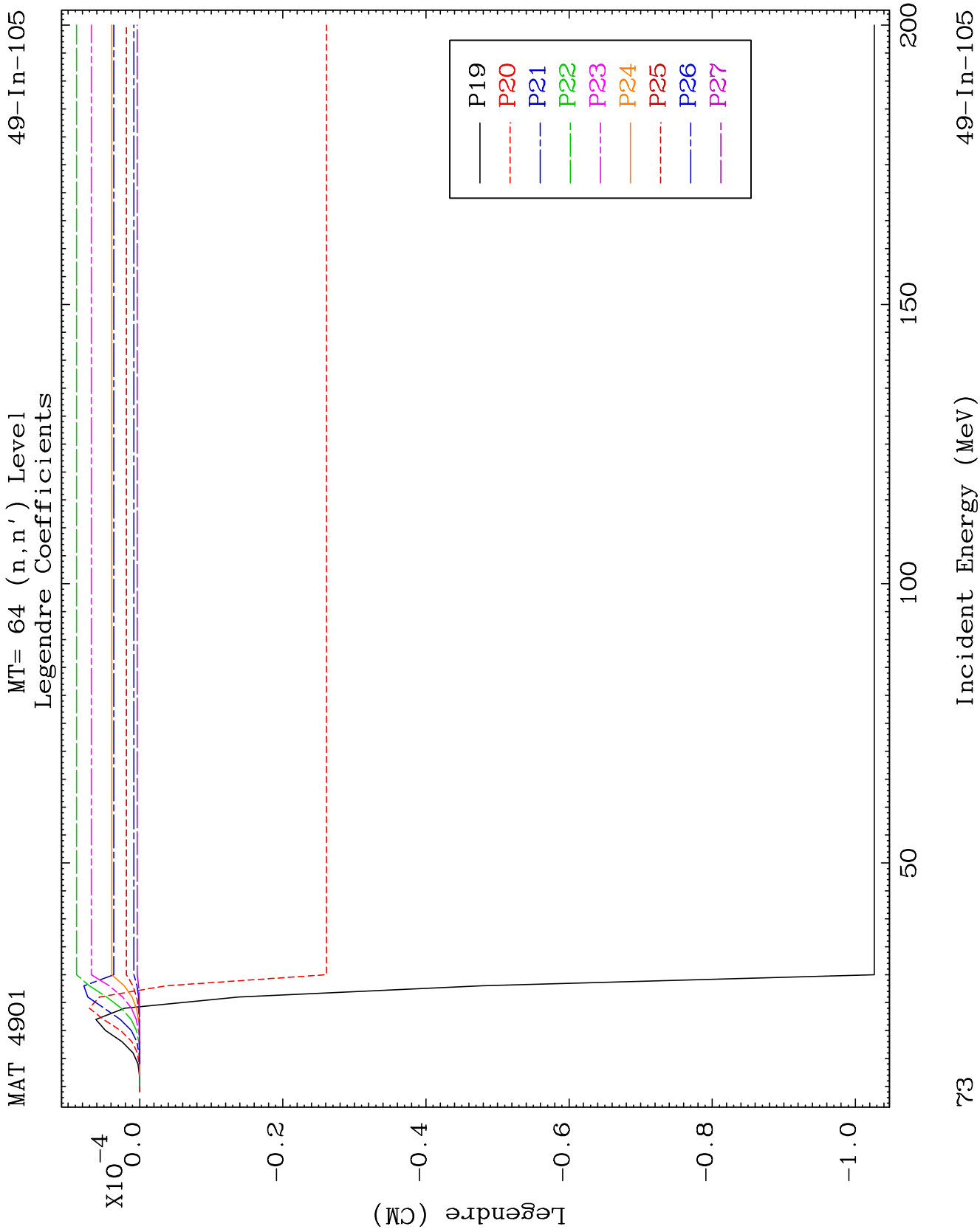


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Incident Energy (MeV)

49-In-105

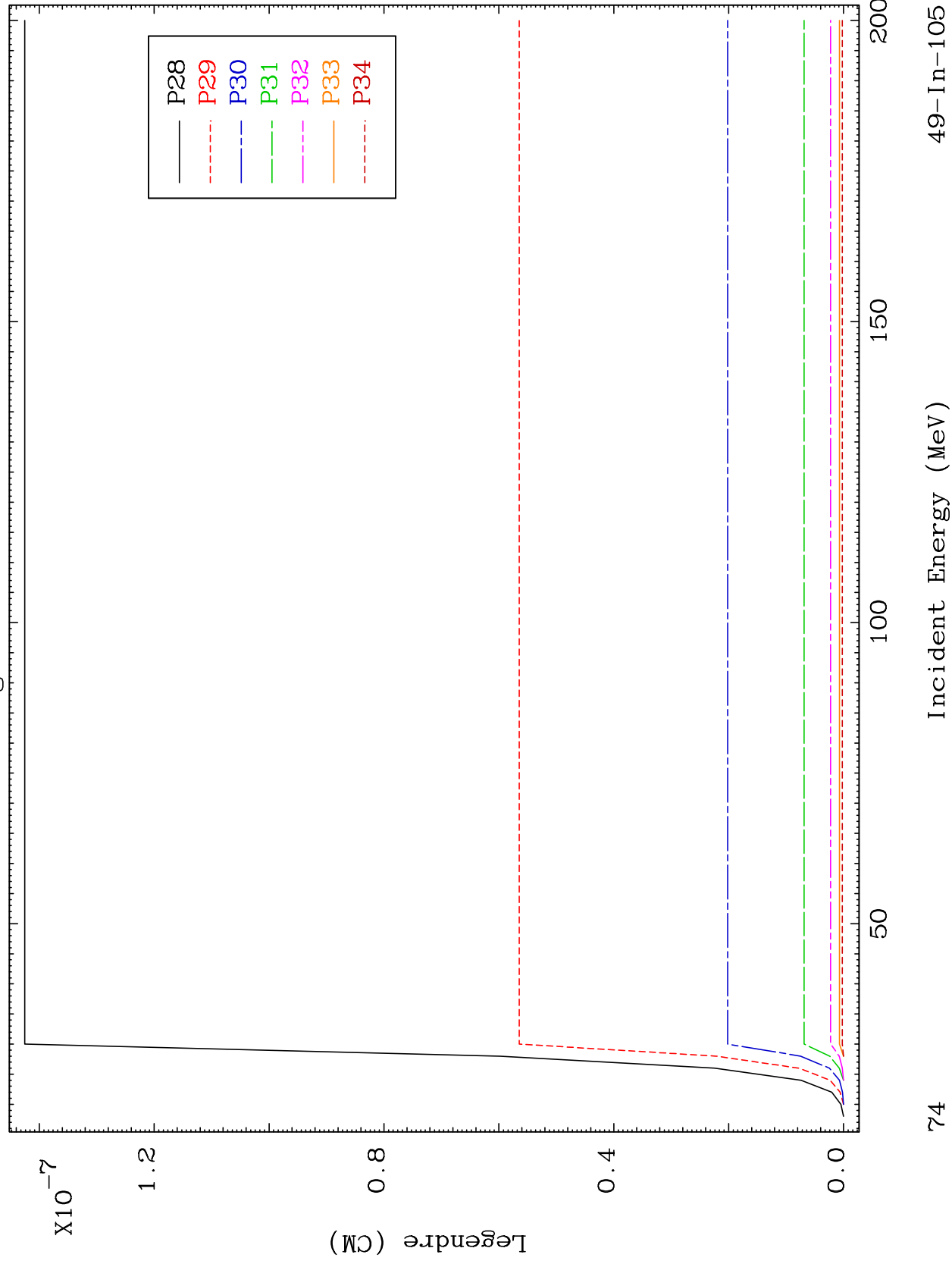


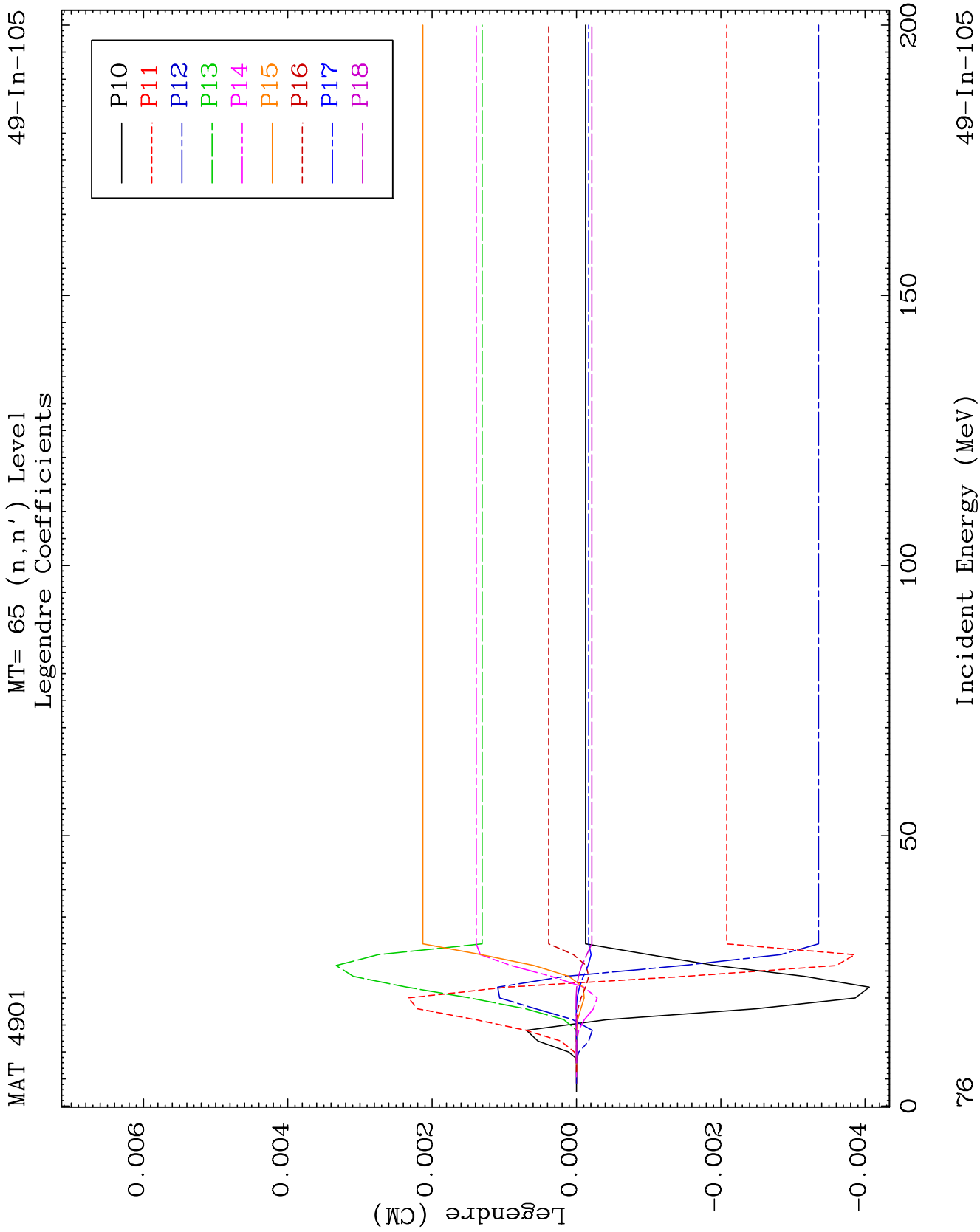


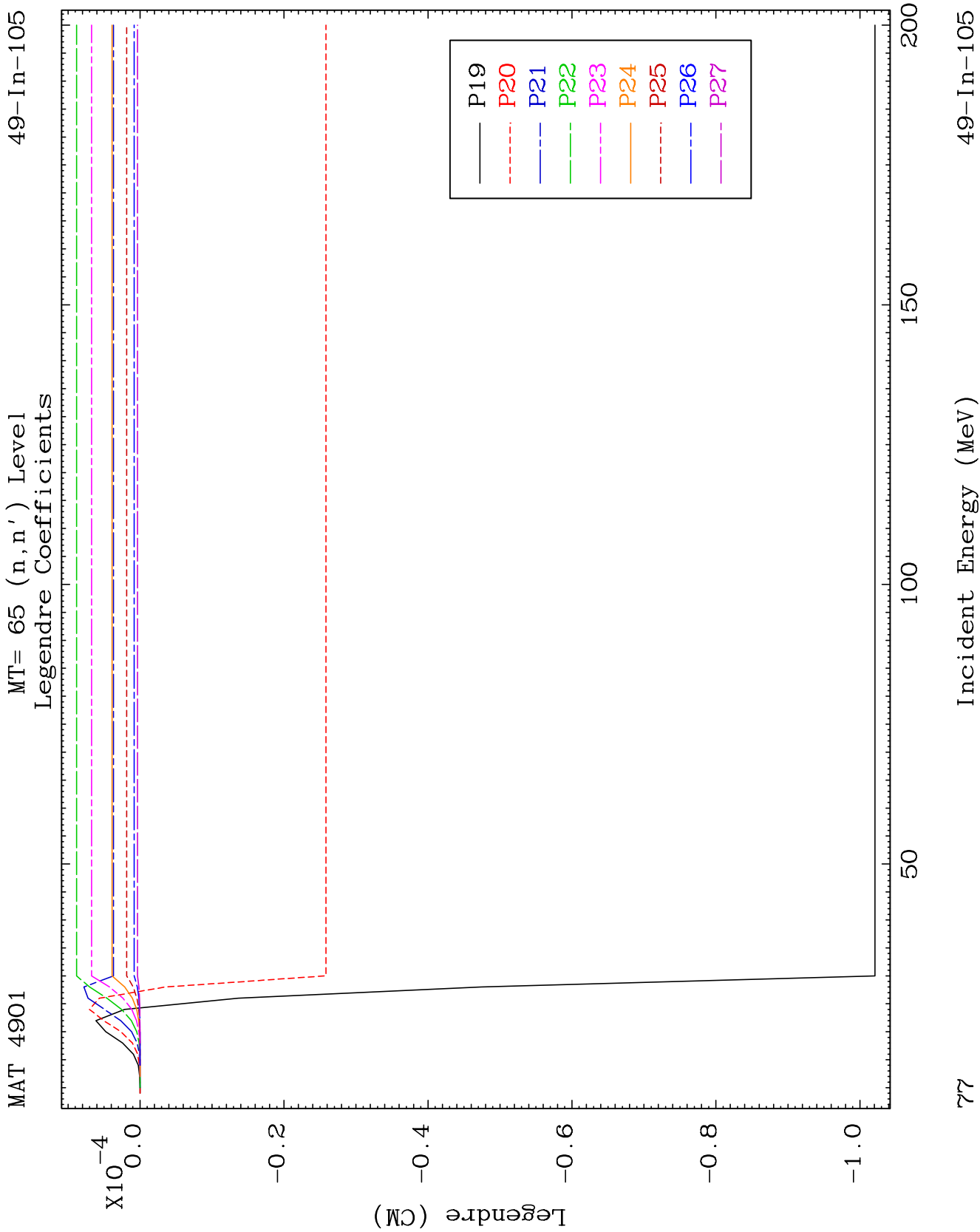
MAT 4901

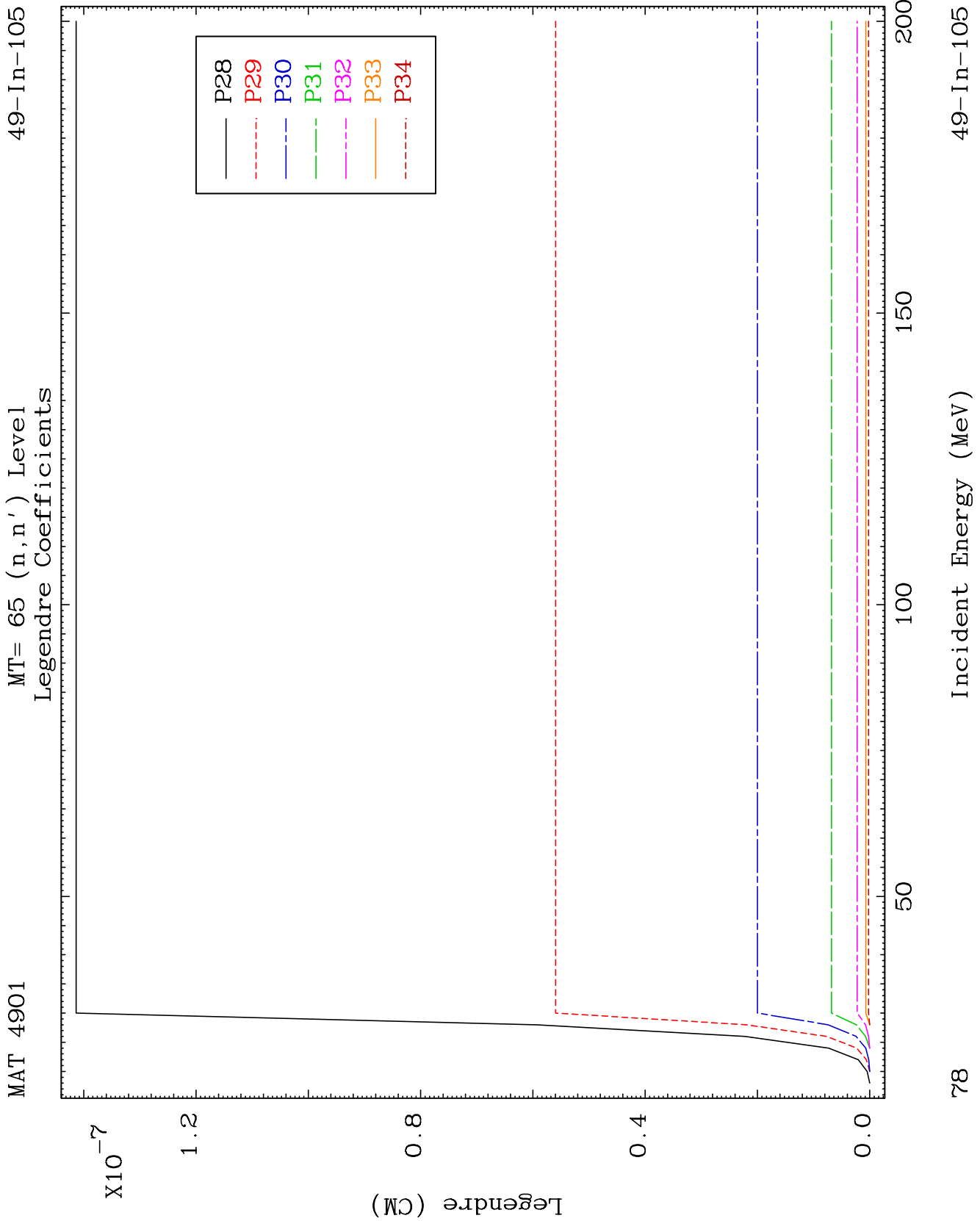
MT= 64 (n, n') Level
Legendre Coefficients

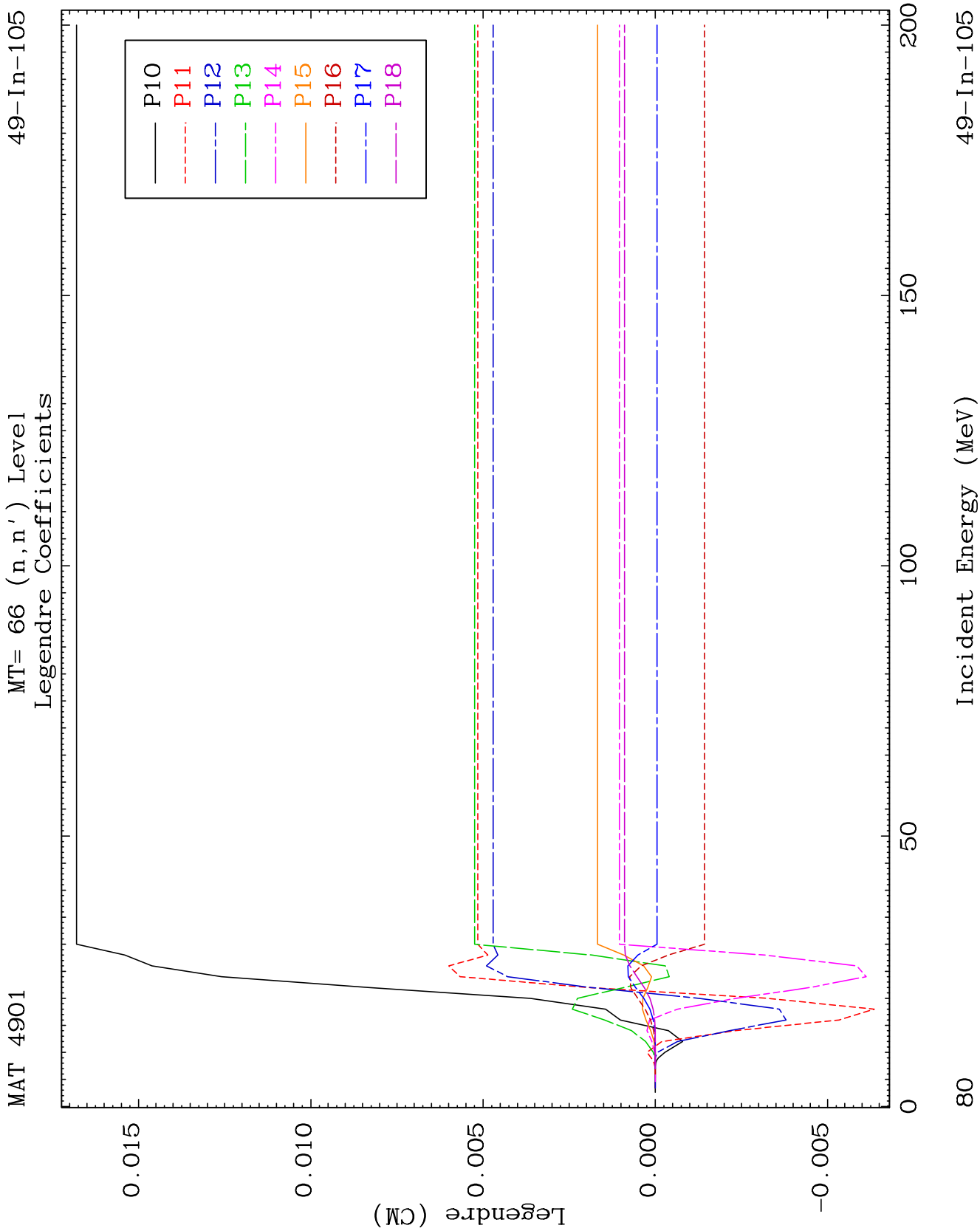
49-In-105

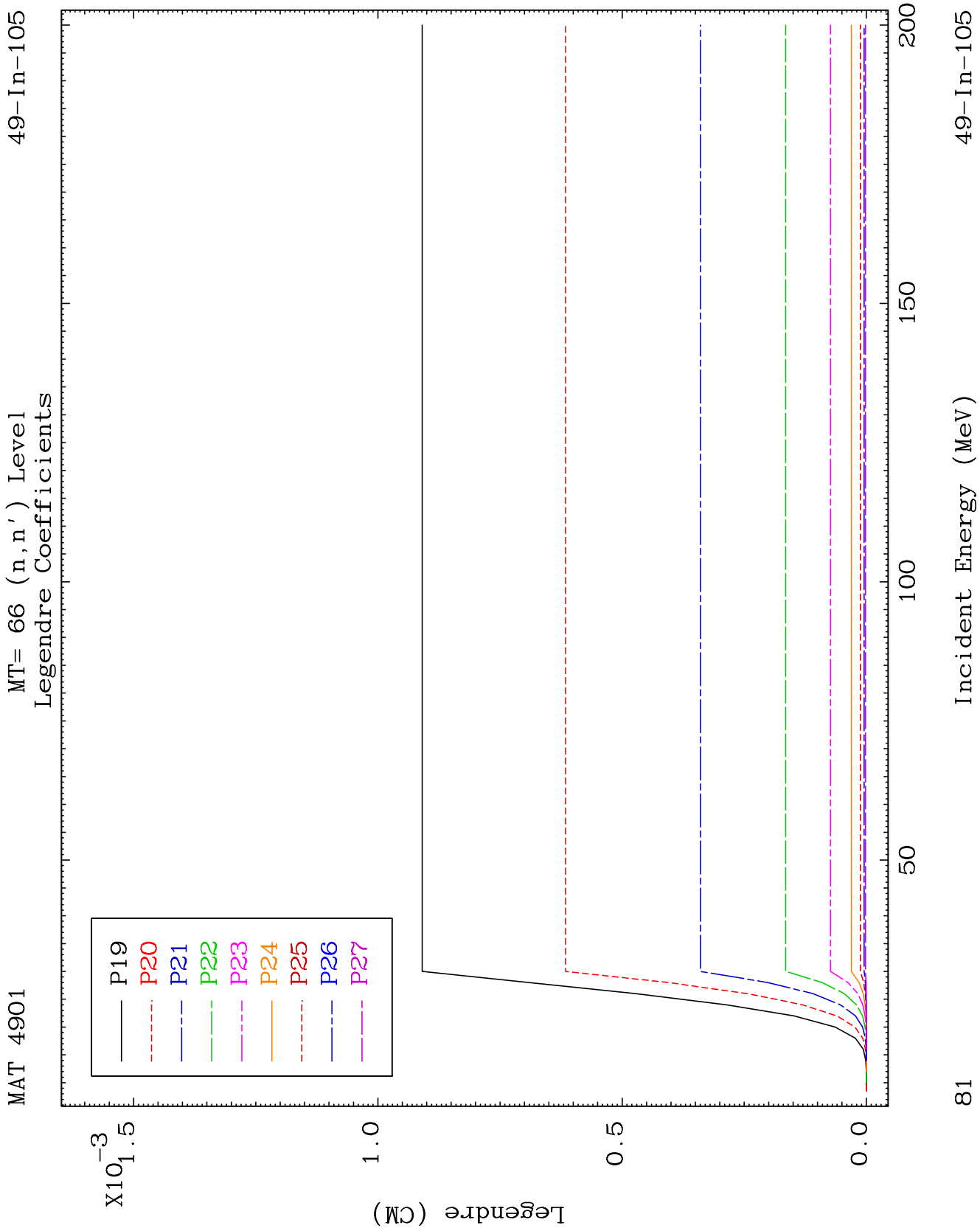


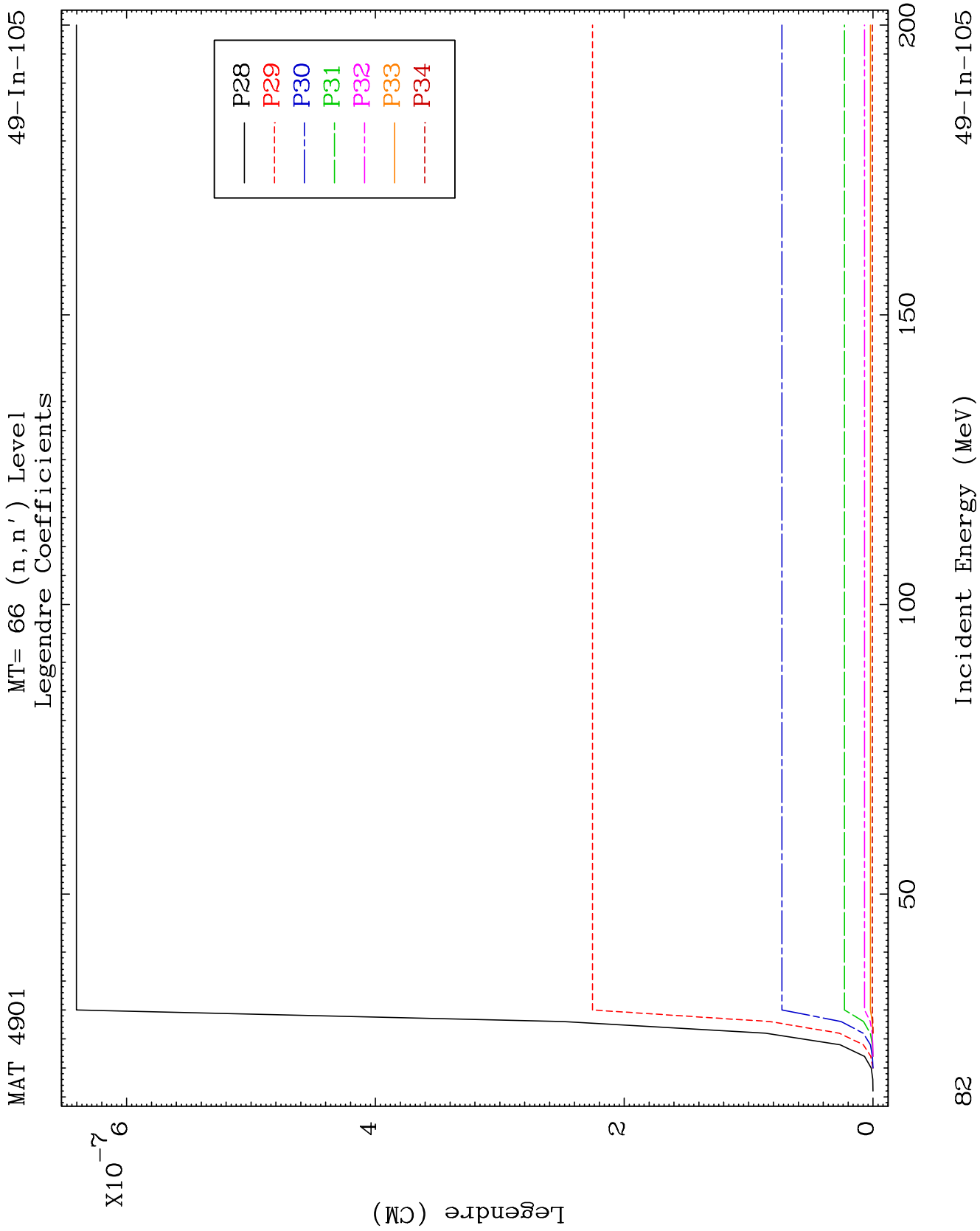


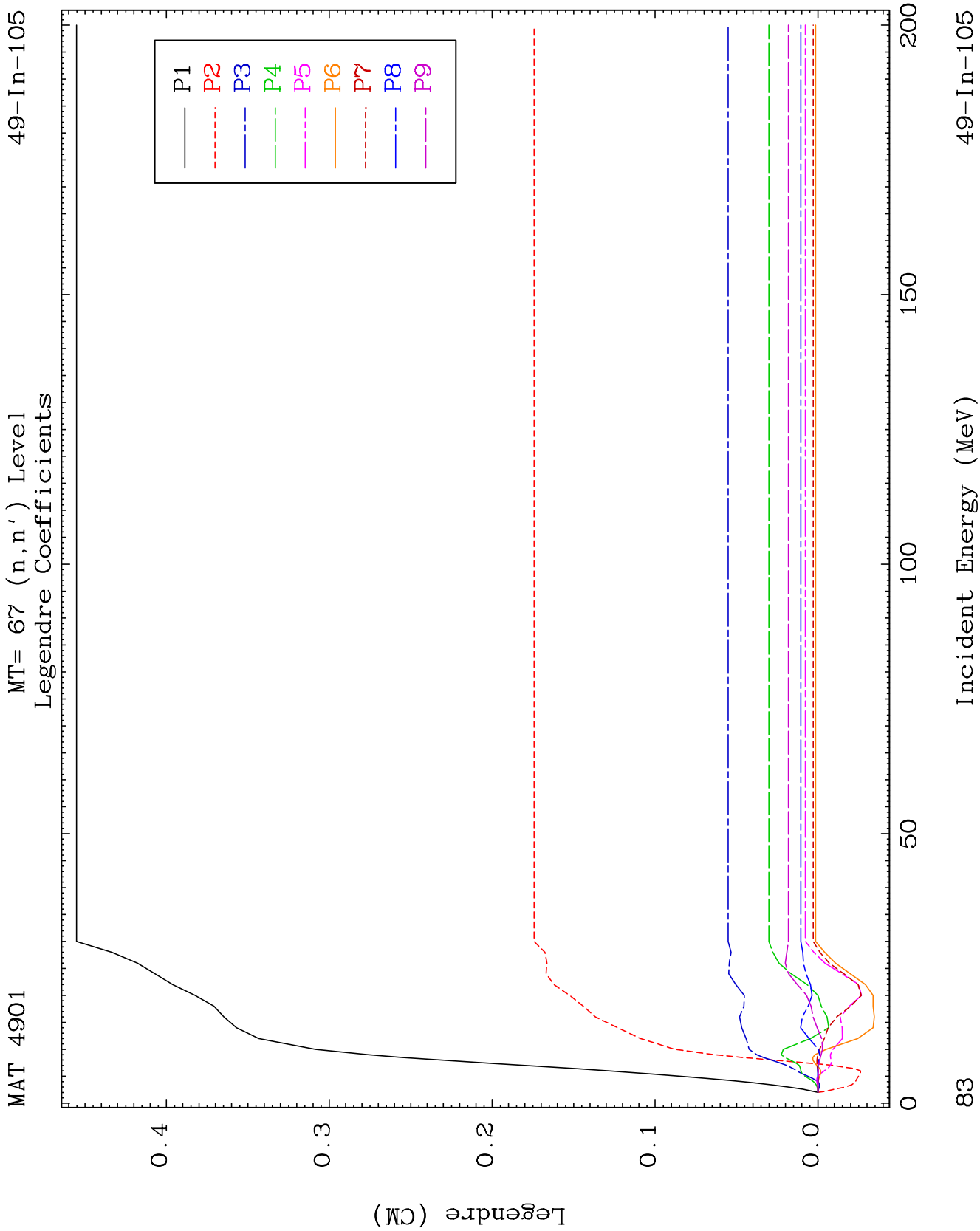


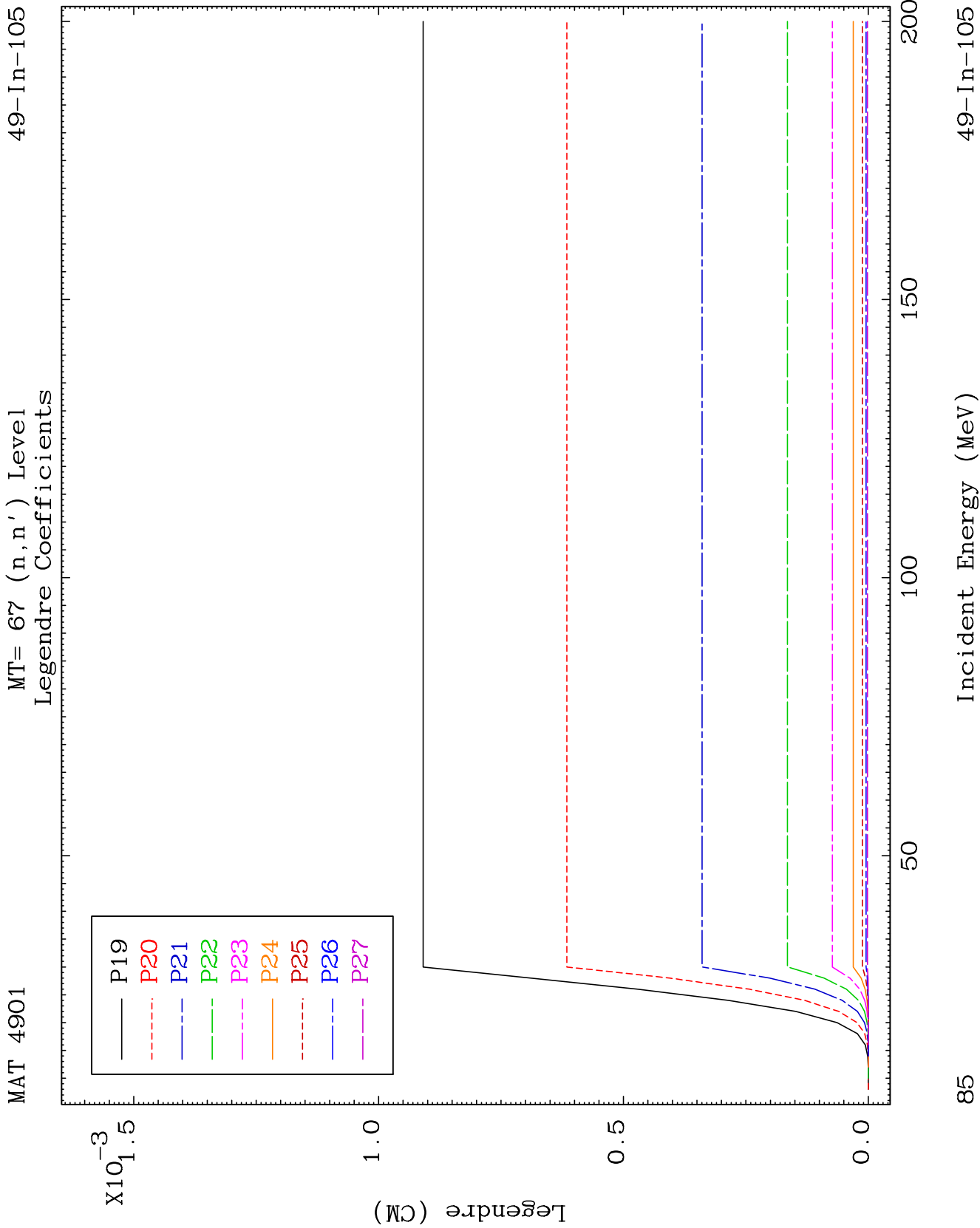


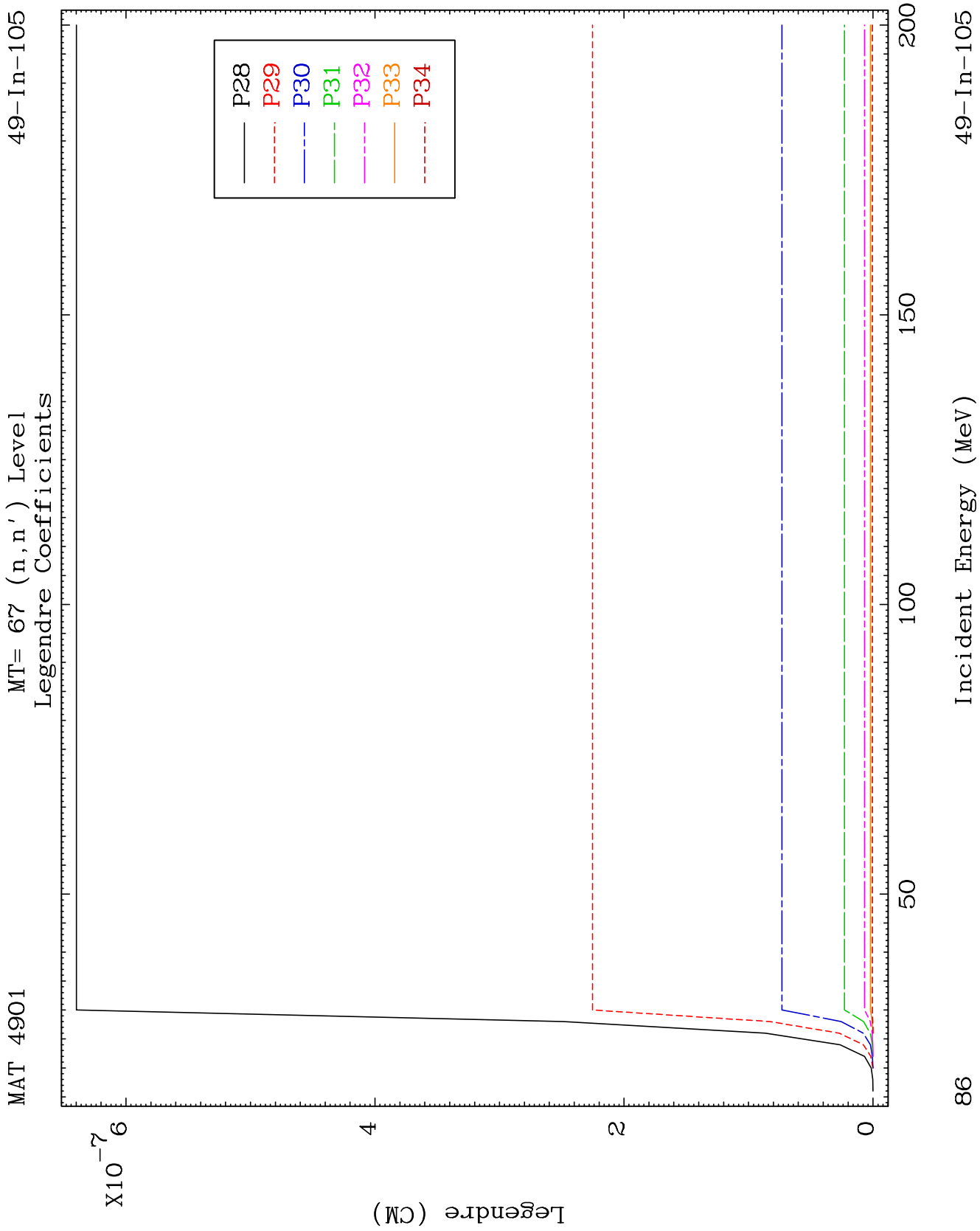


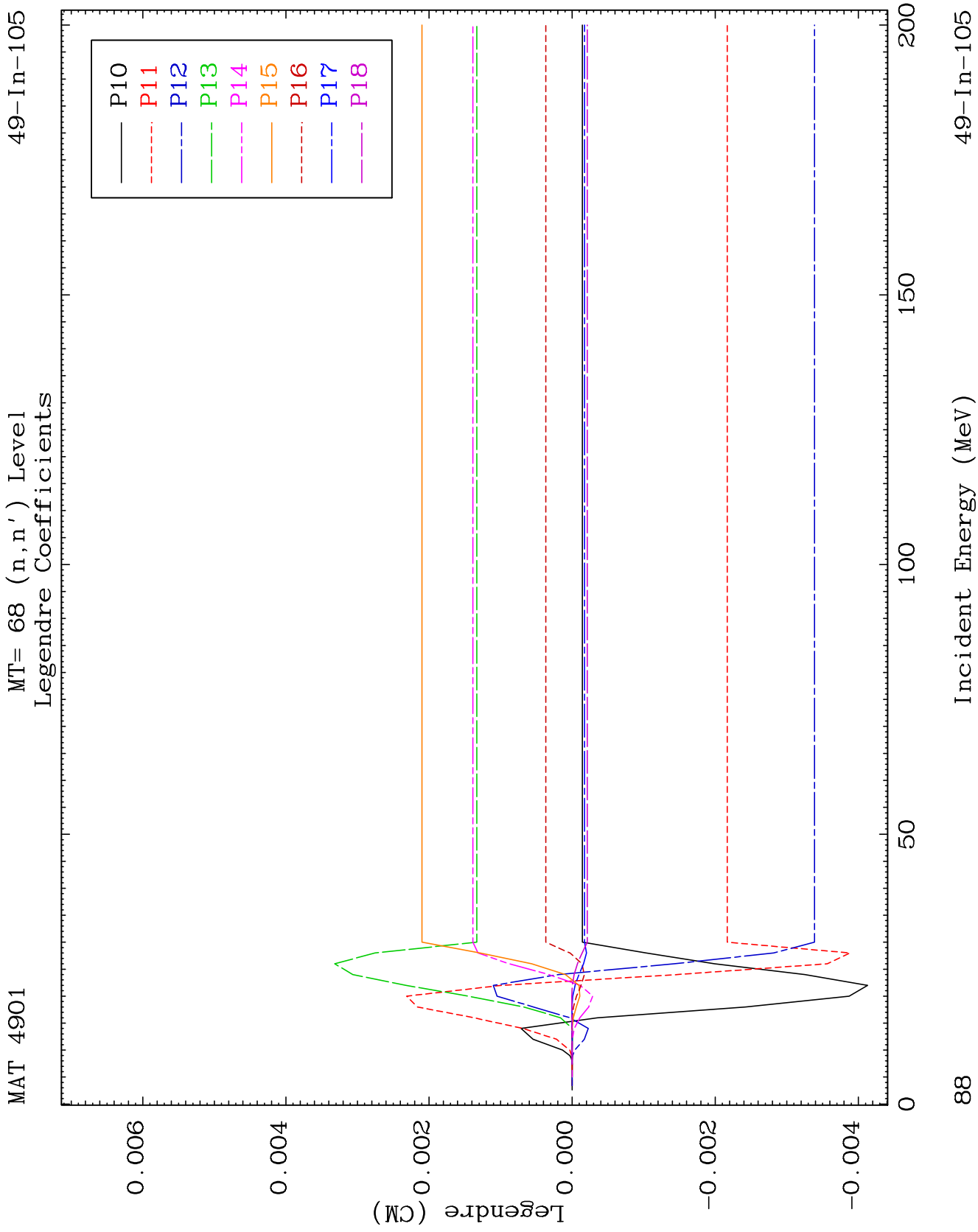


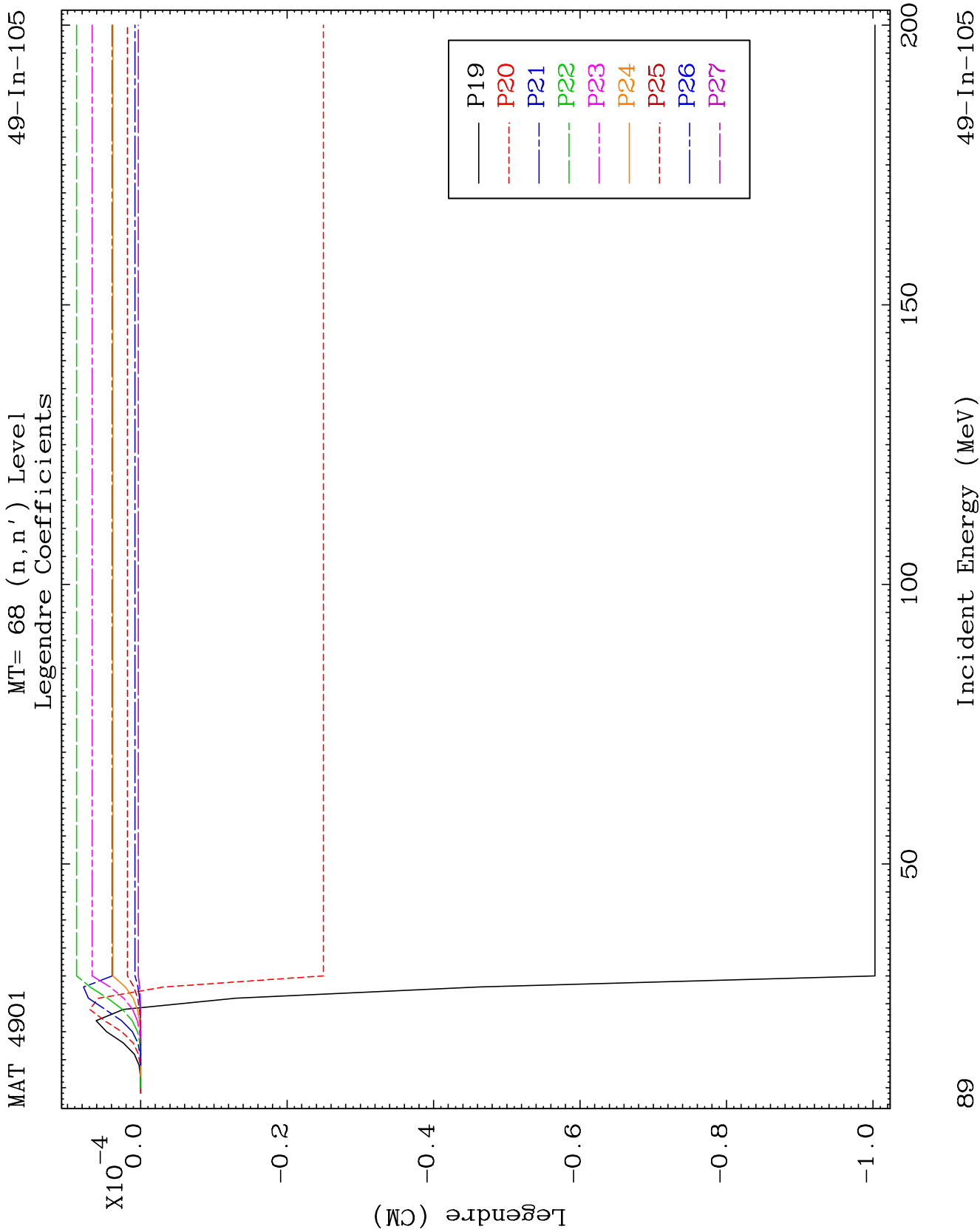


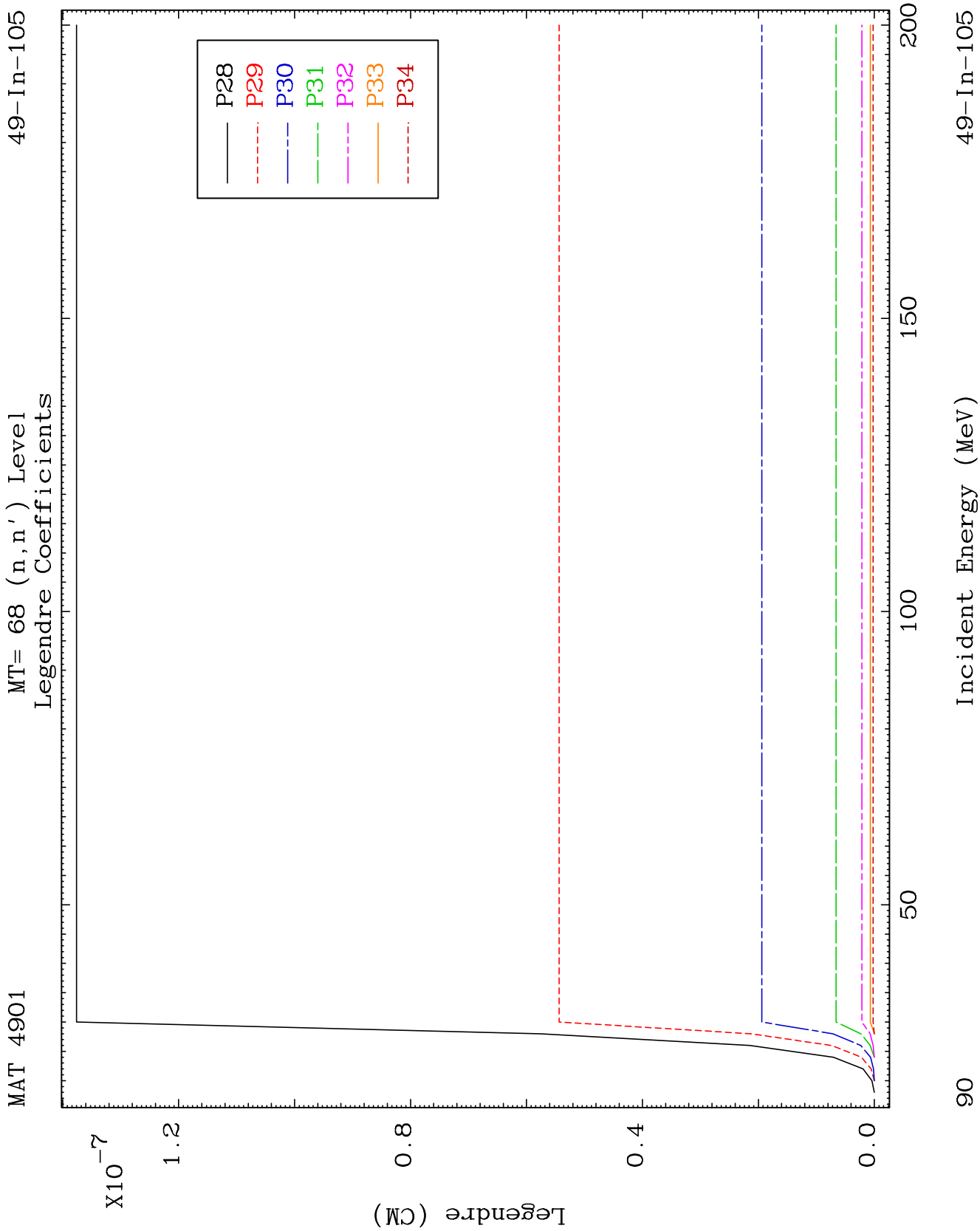


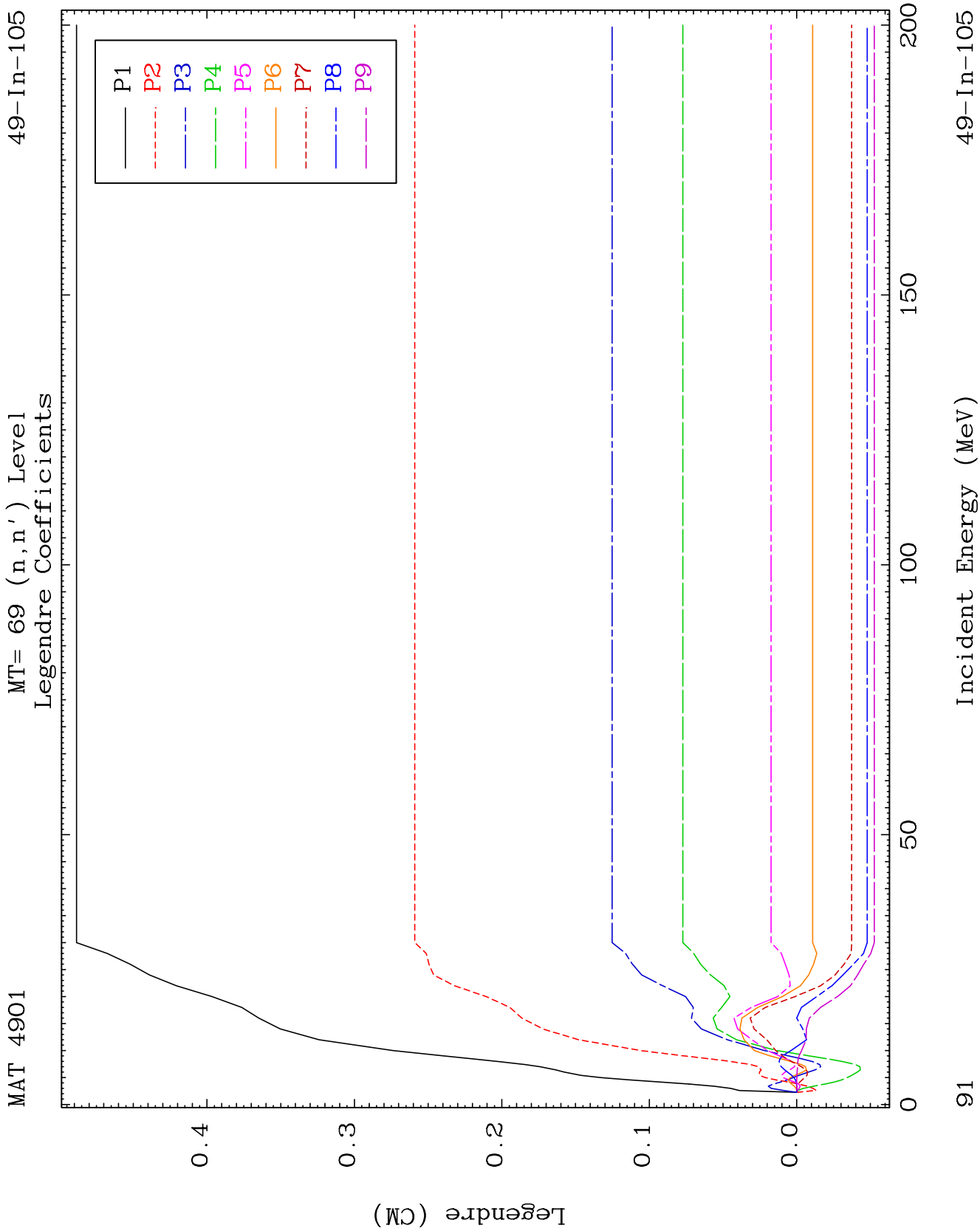


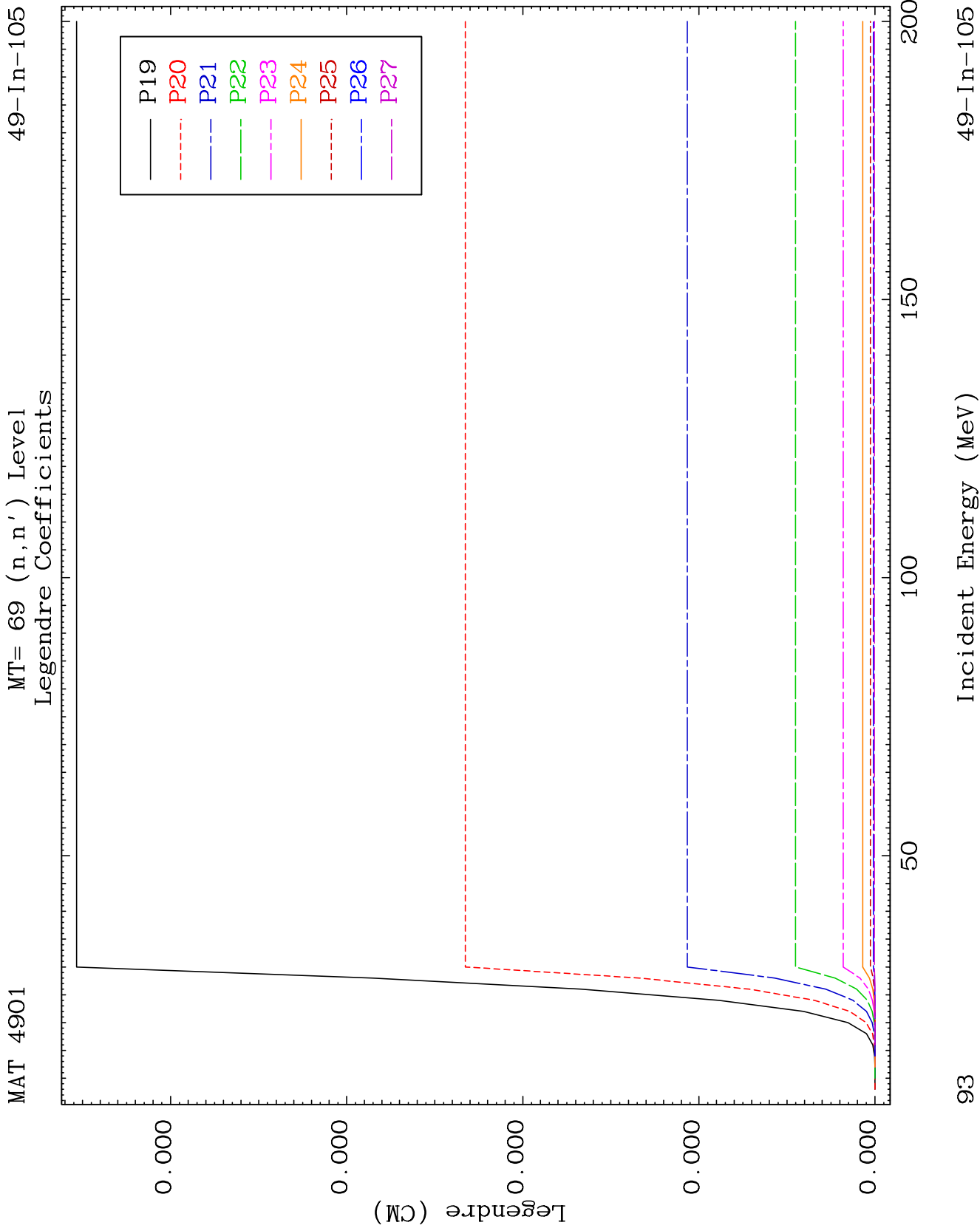








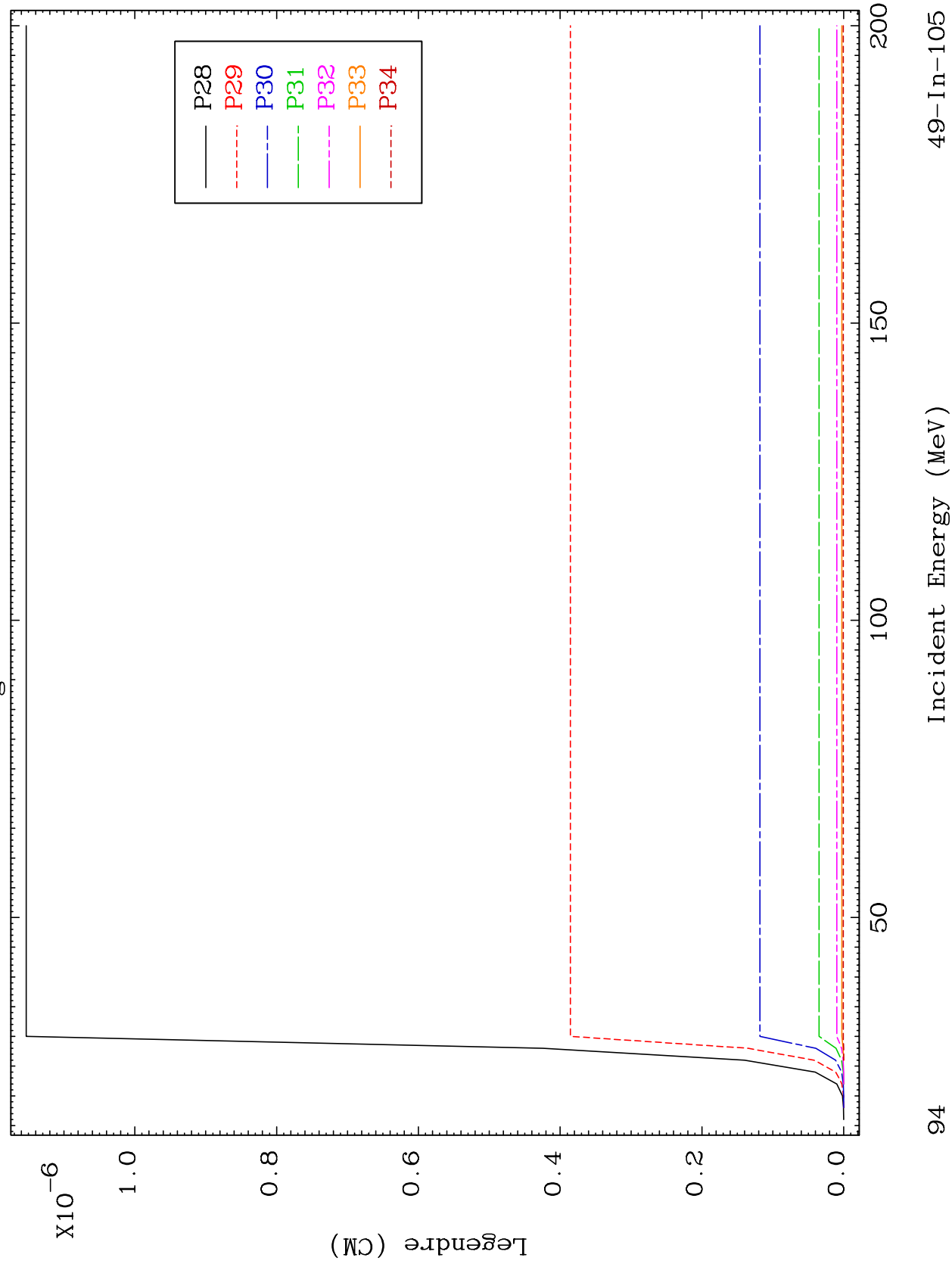


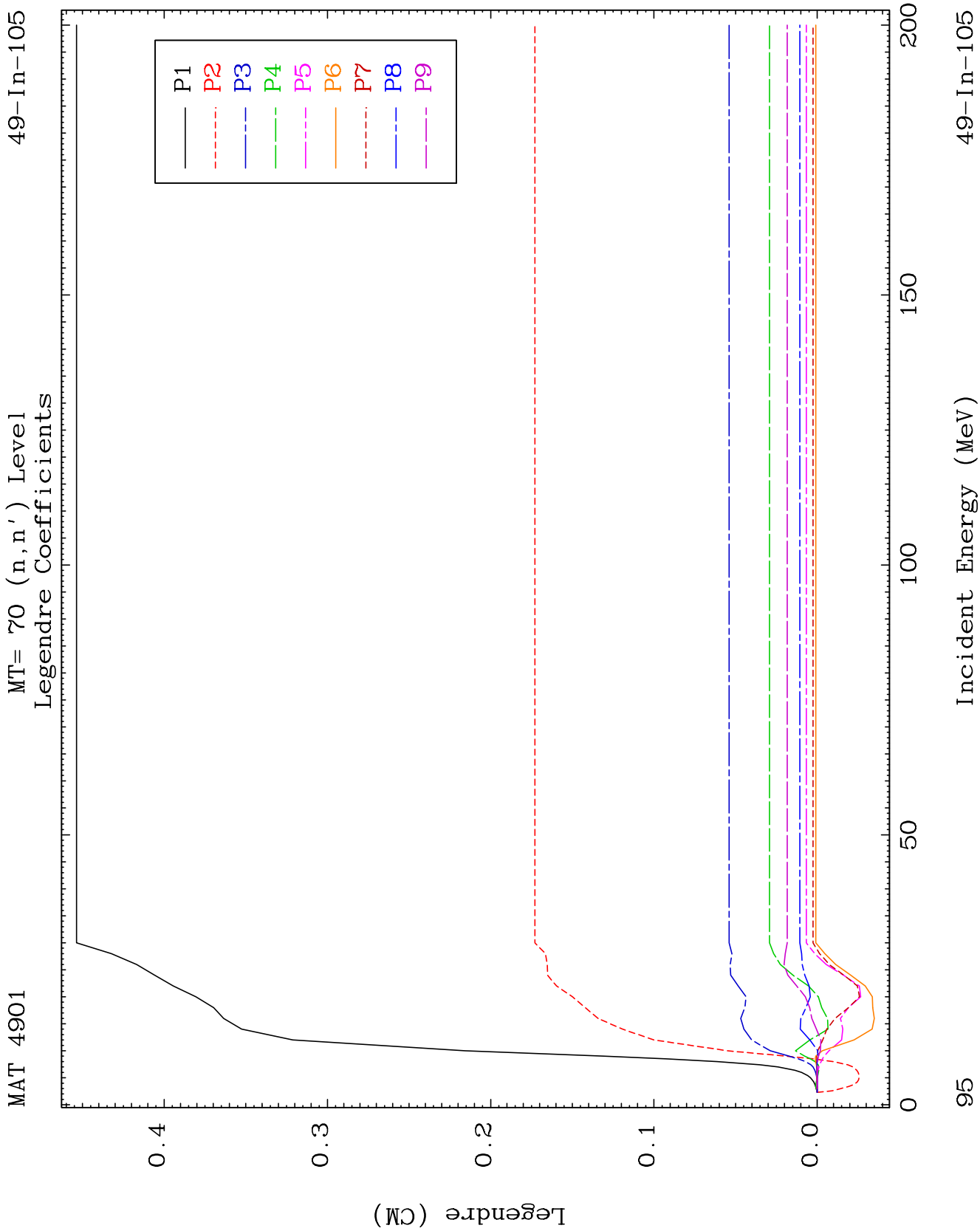


MAT 4901

MT= 69 (n, n') Level
Legendre Coefficients

49-In-105

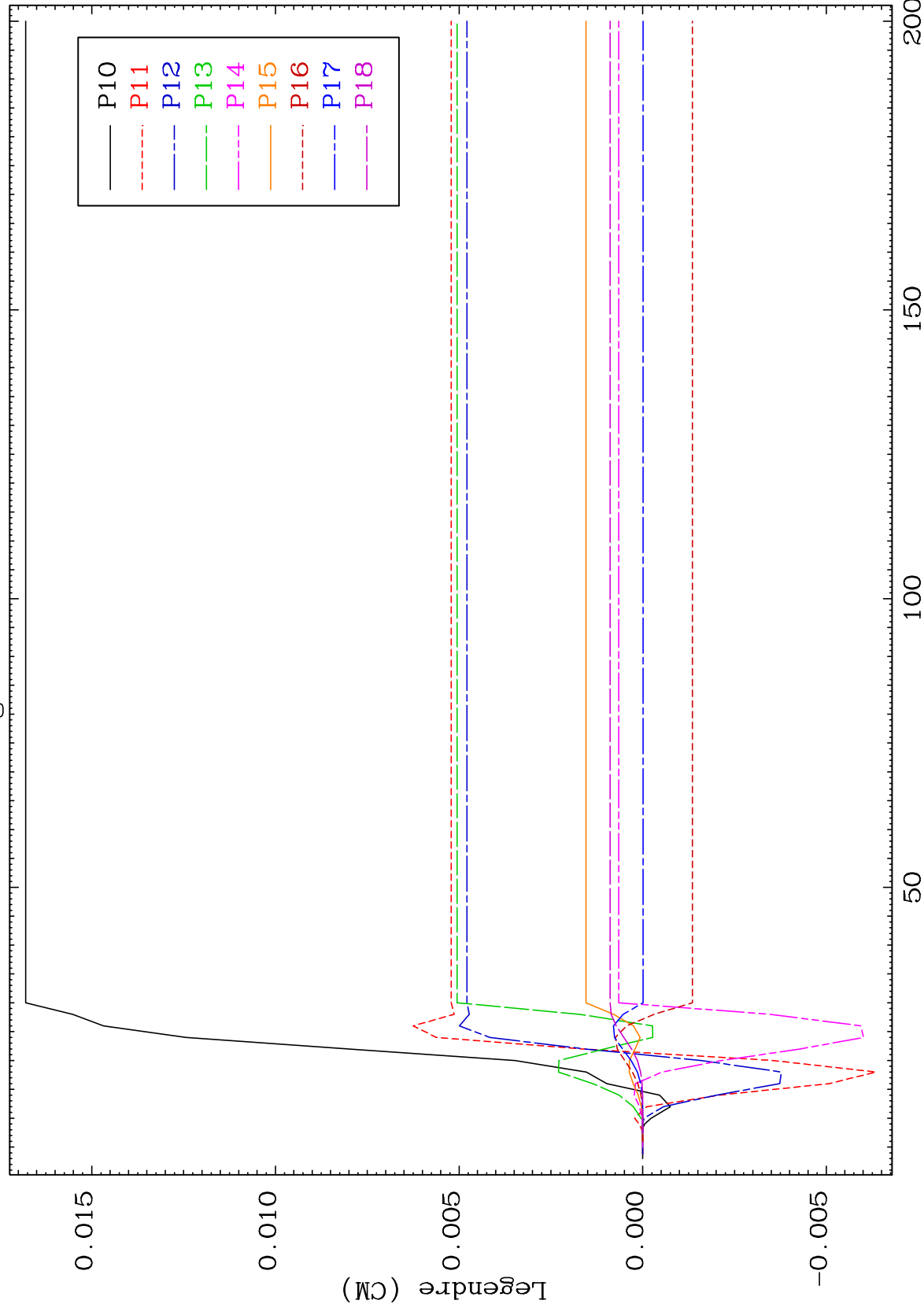




MAT 4901

MT= 70 (n, n') Level
Legendre Coefficients

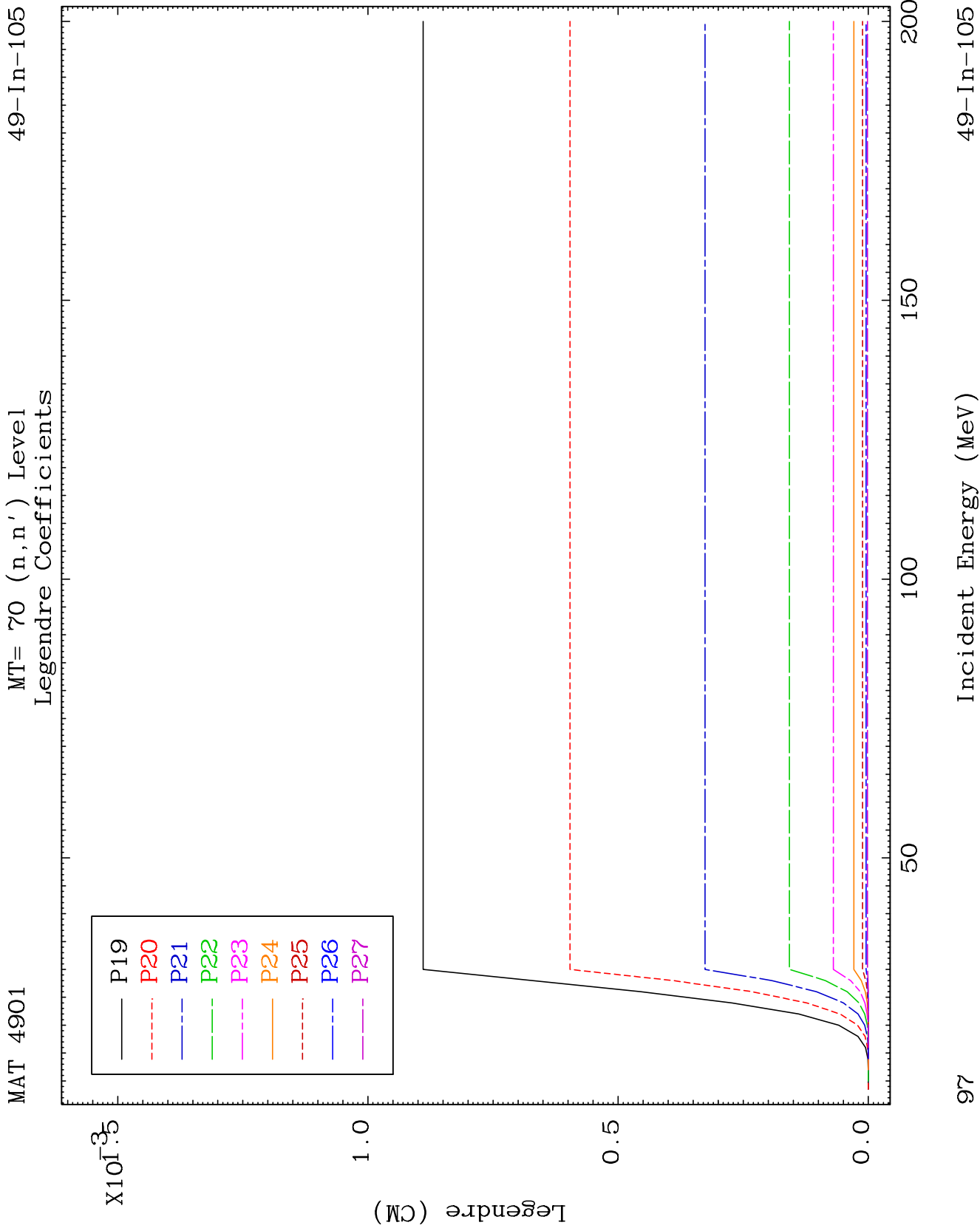
49-In-105

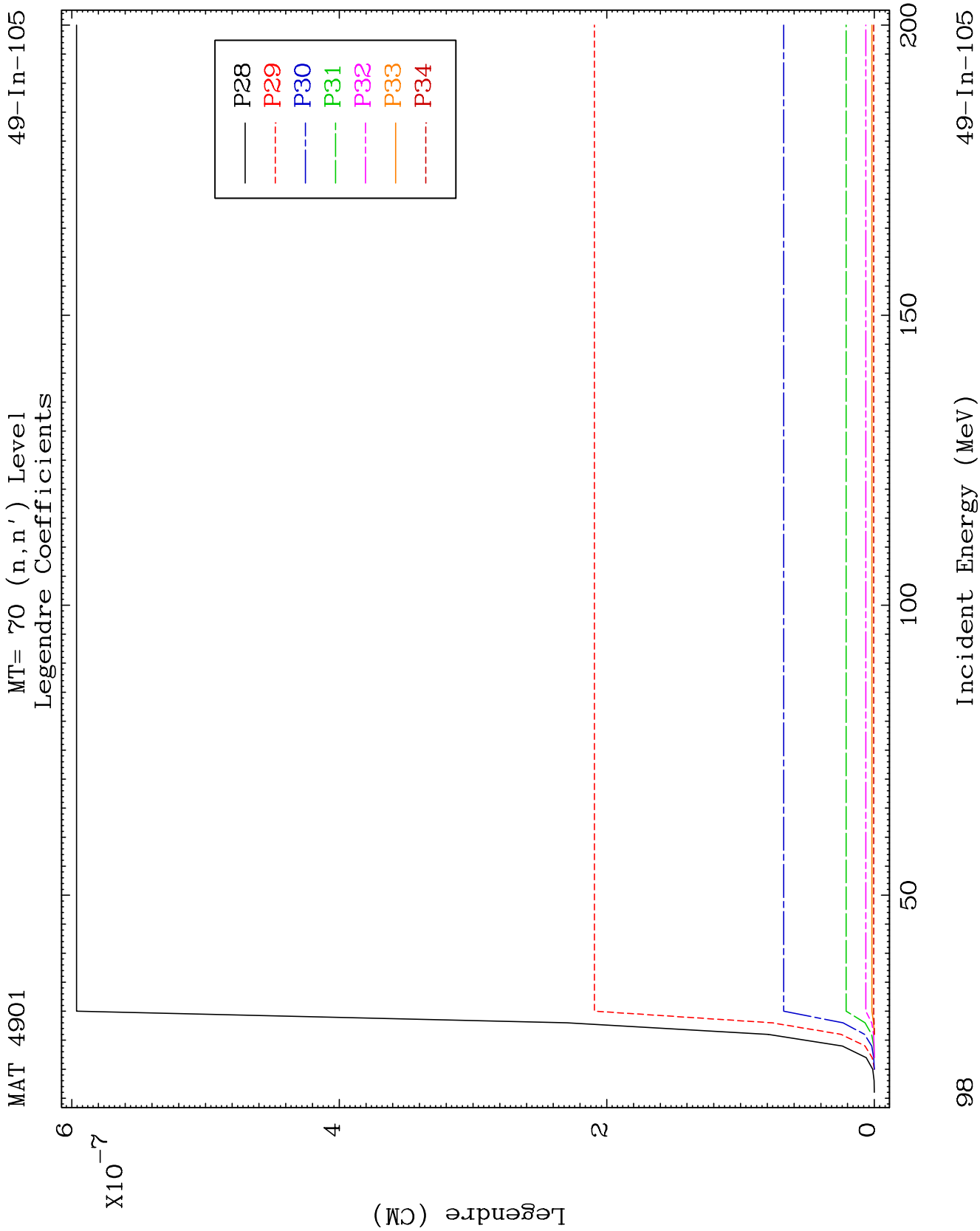


99

Incident Energy (MeV)

49-In-105

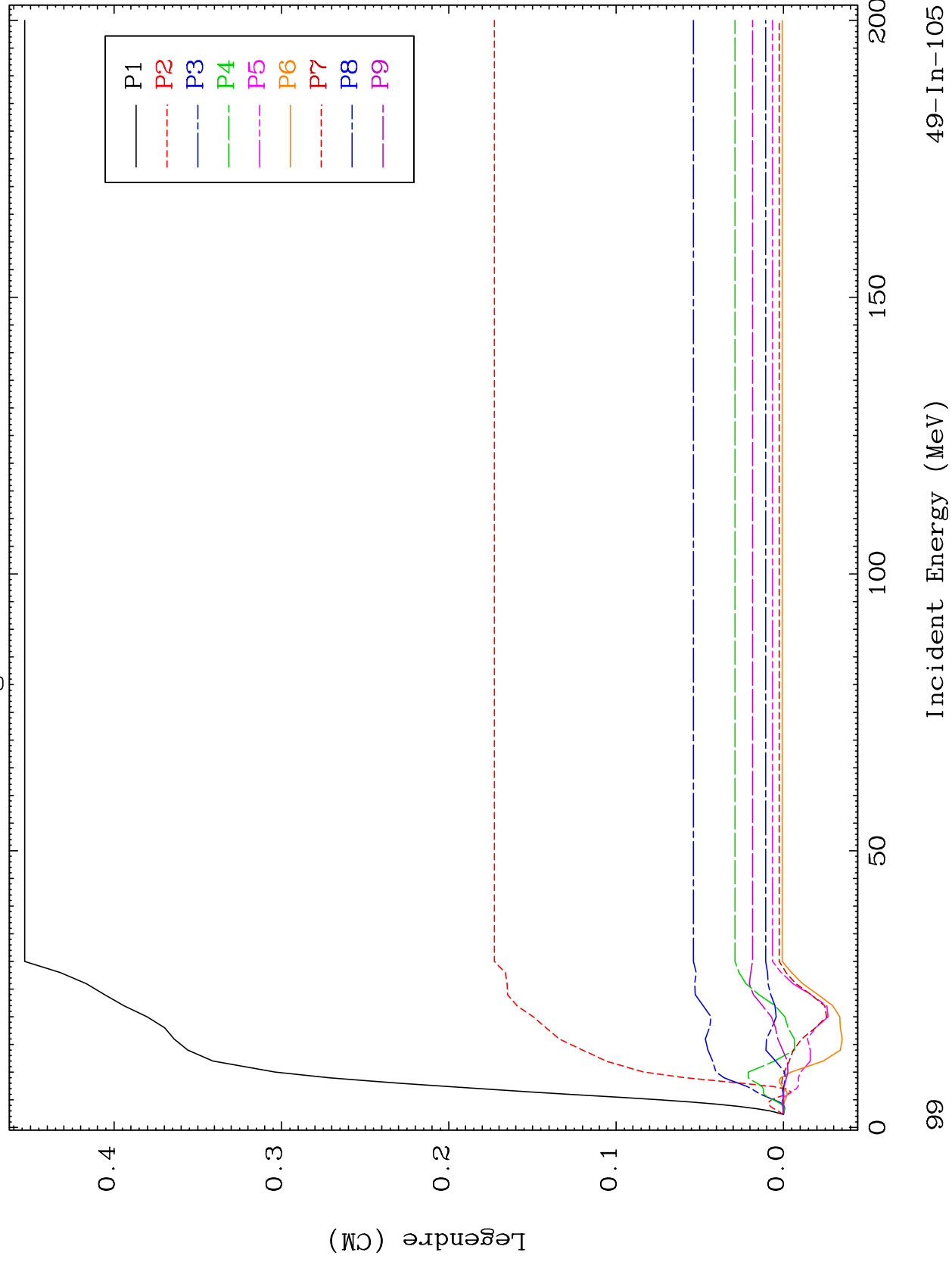


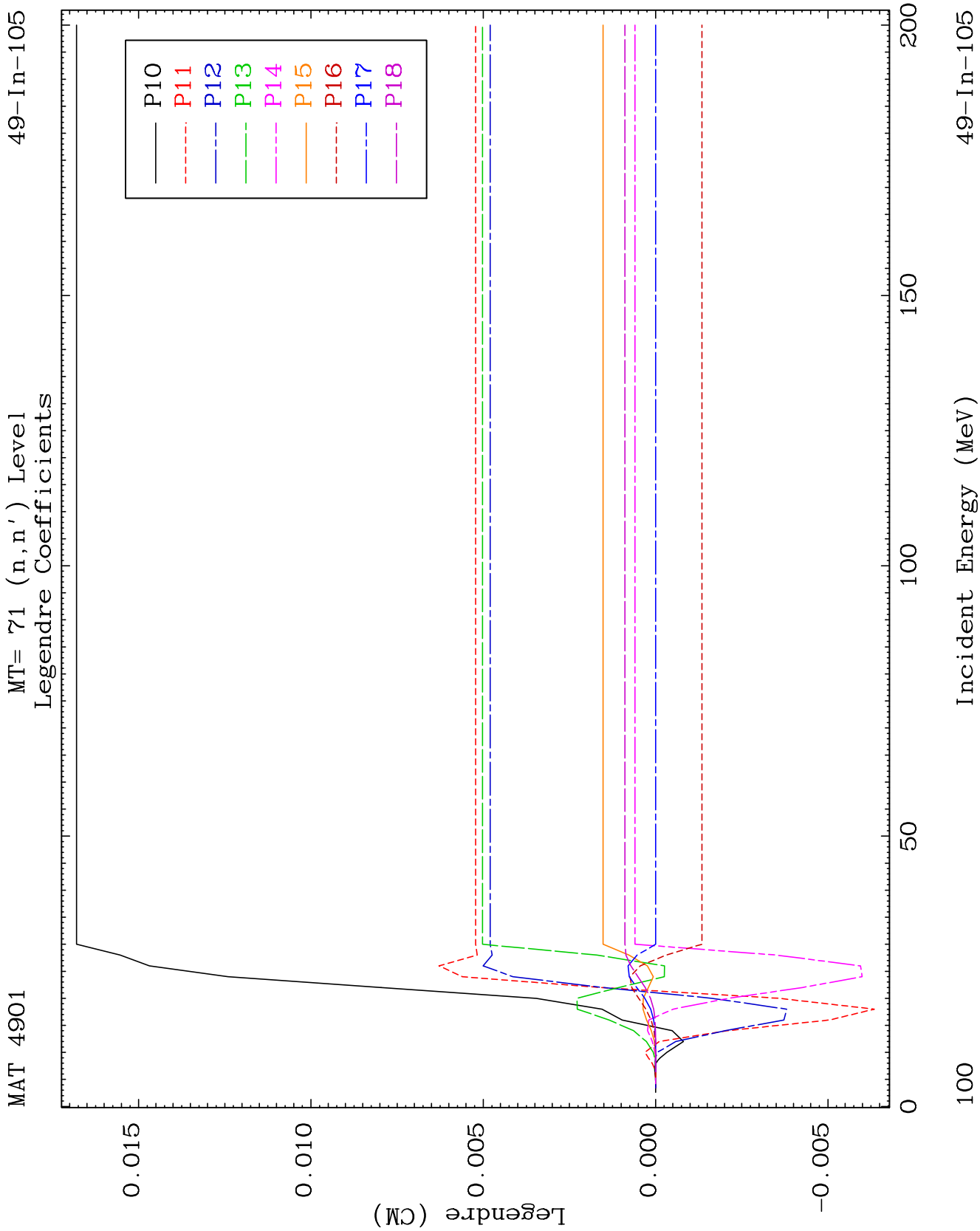


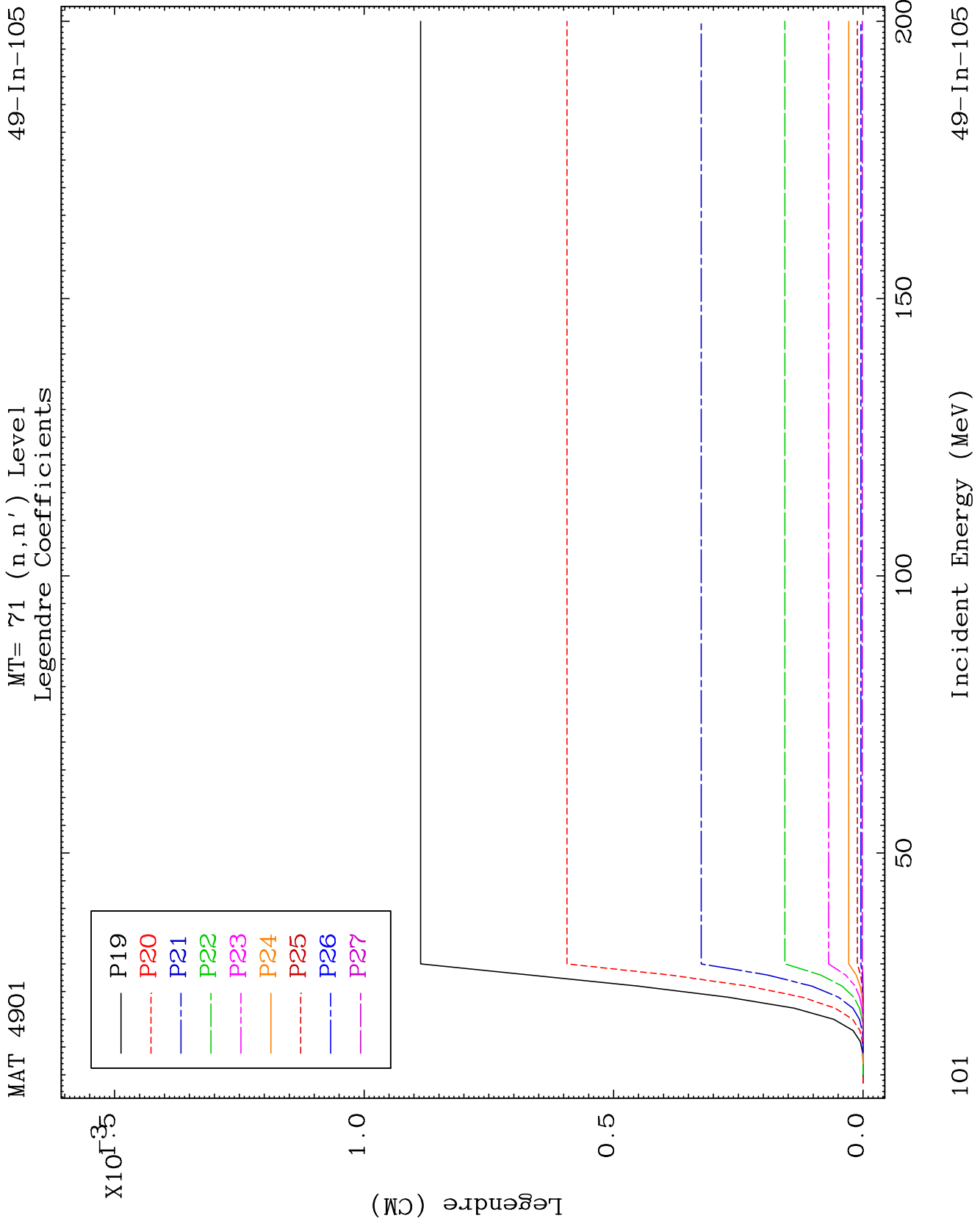
MAT 4901

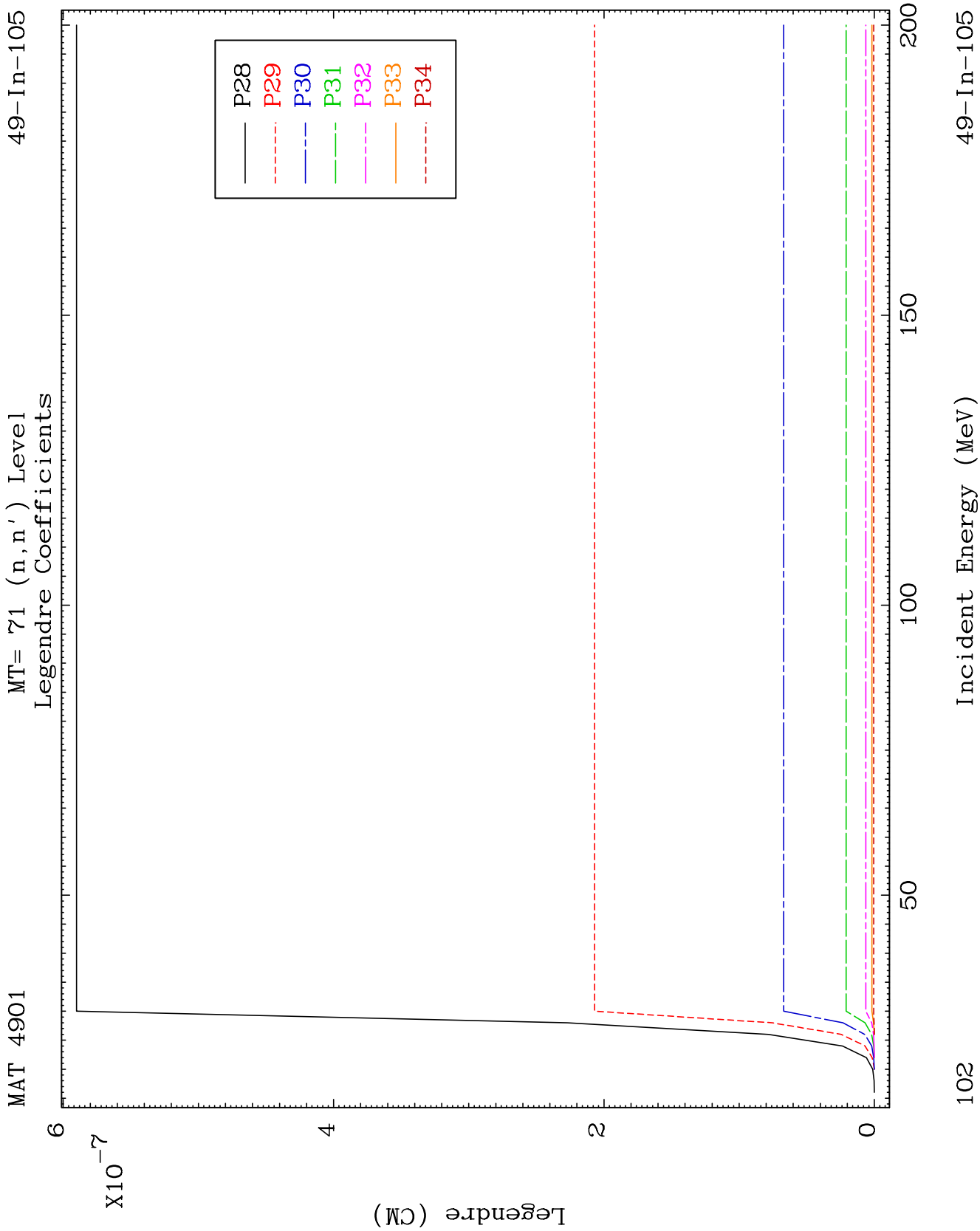
MT= 71 (n, n') Level
Legendre Coefficients

49-In-105





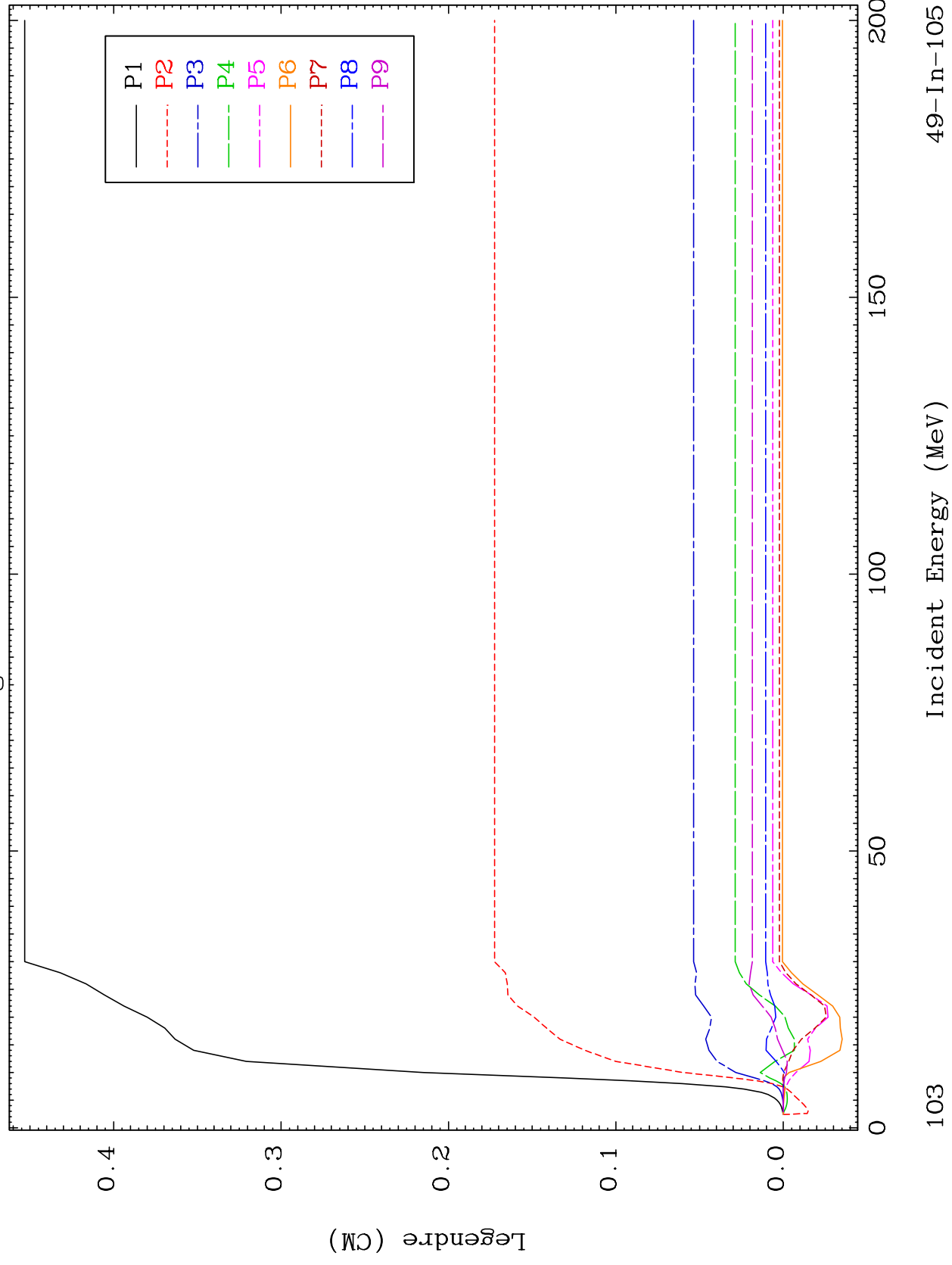




MAT 4901

MT= 72 (n, n') Level
Legendre Coefficients

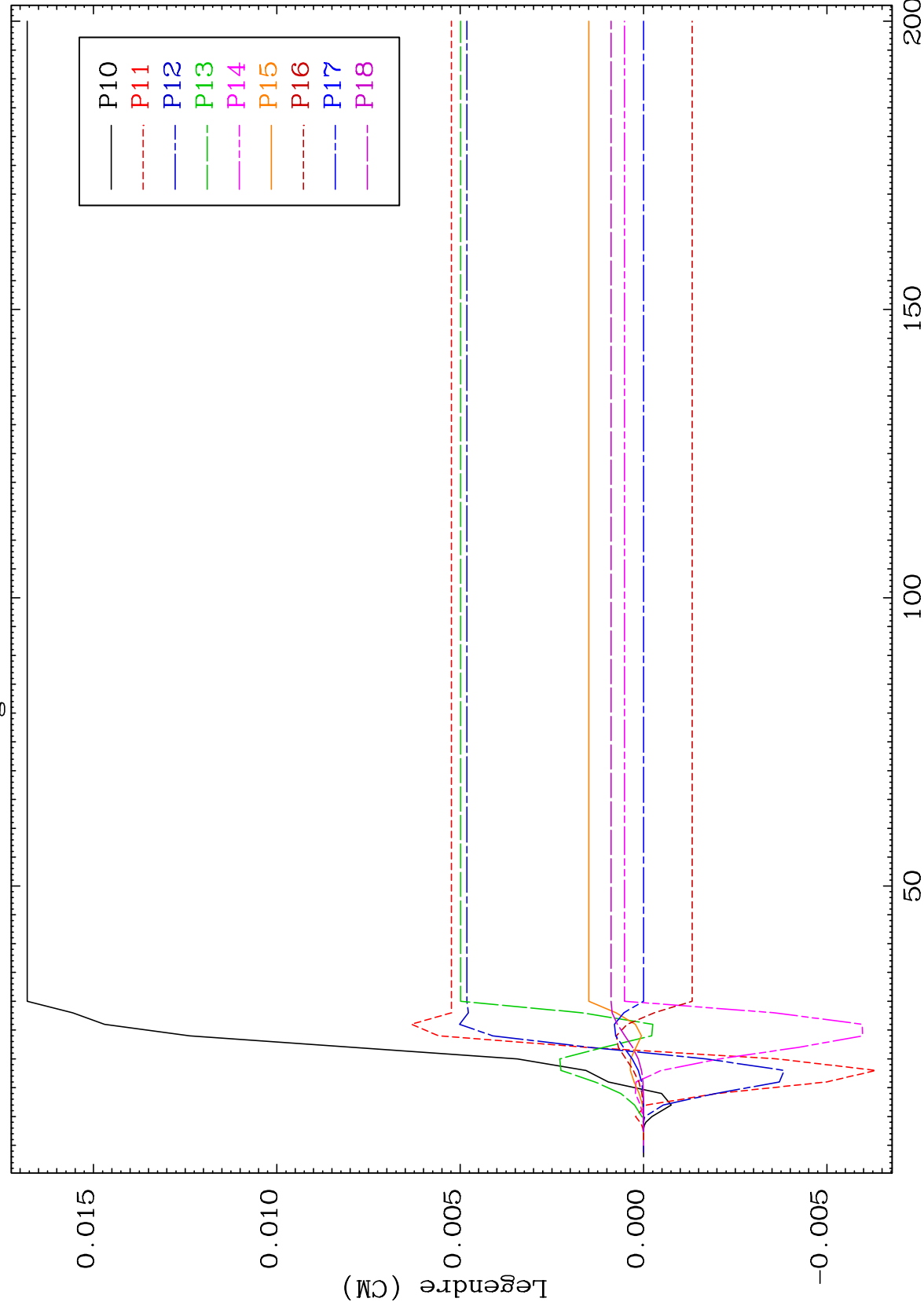
49-In-105

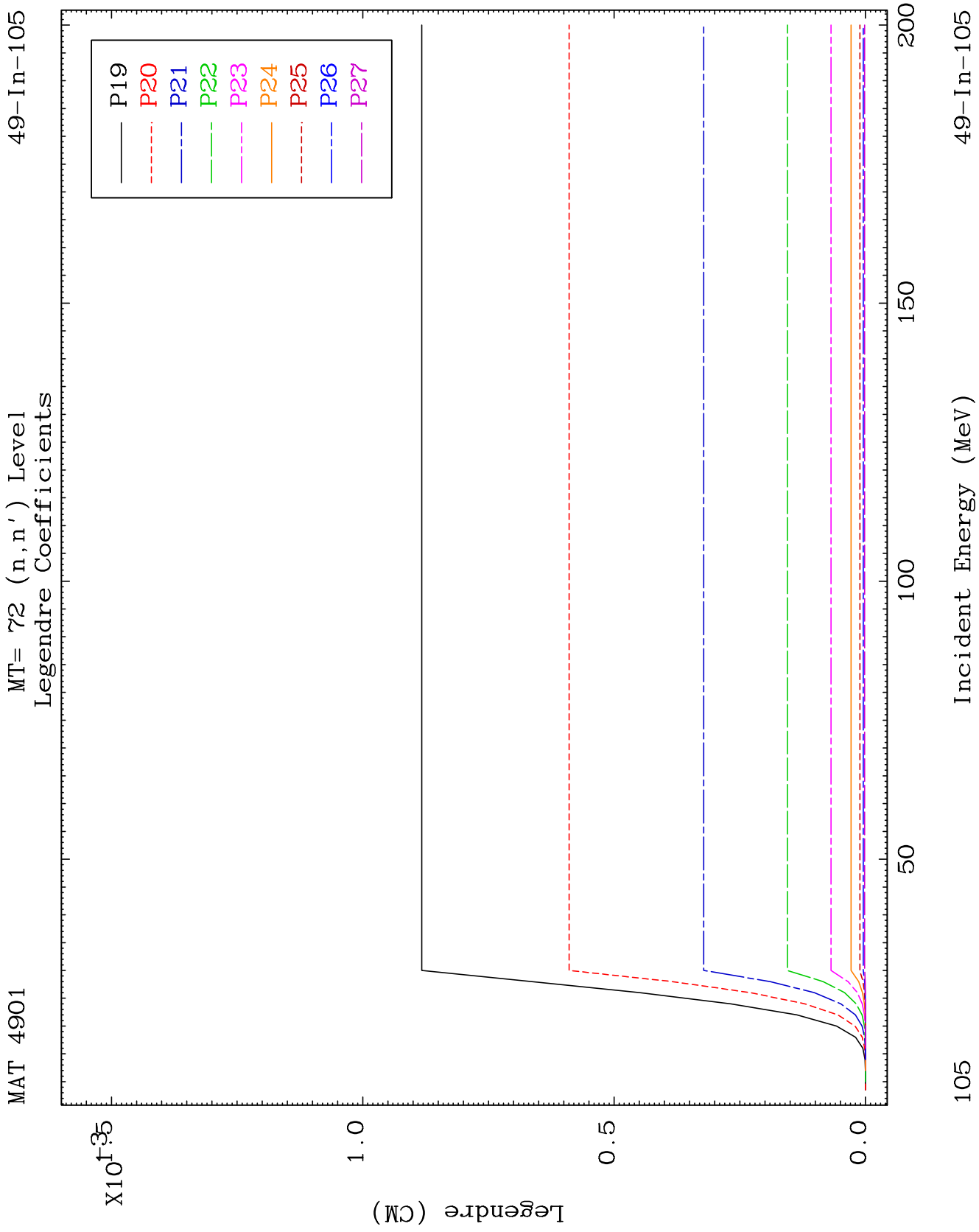


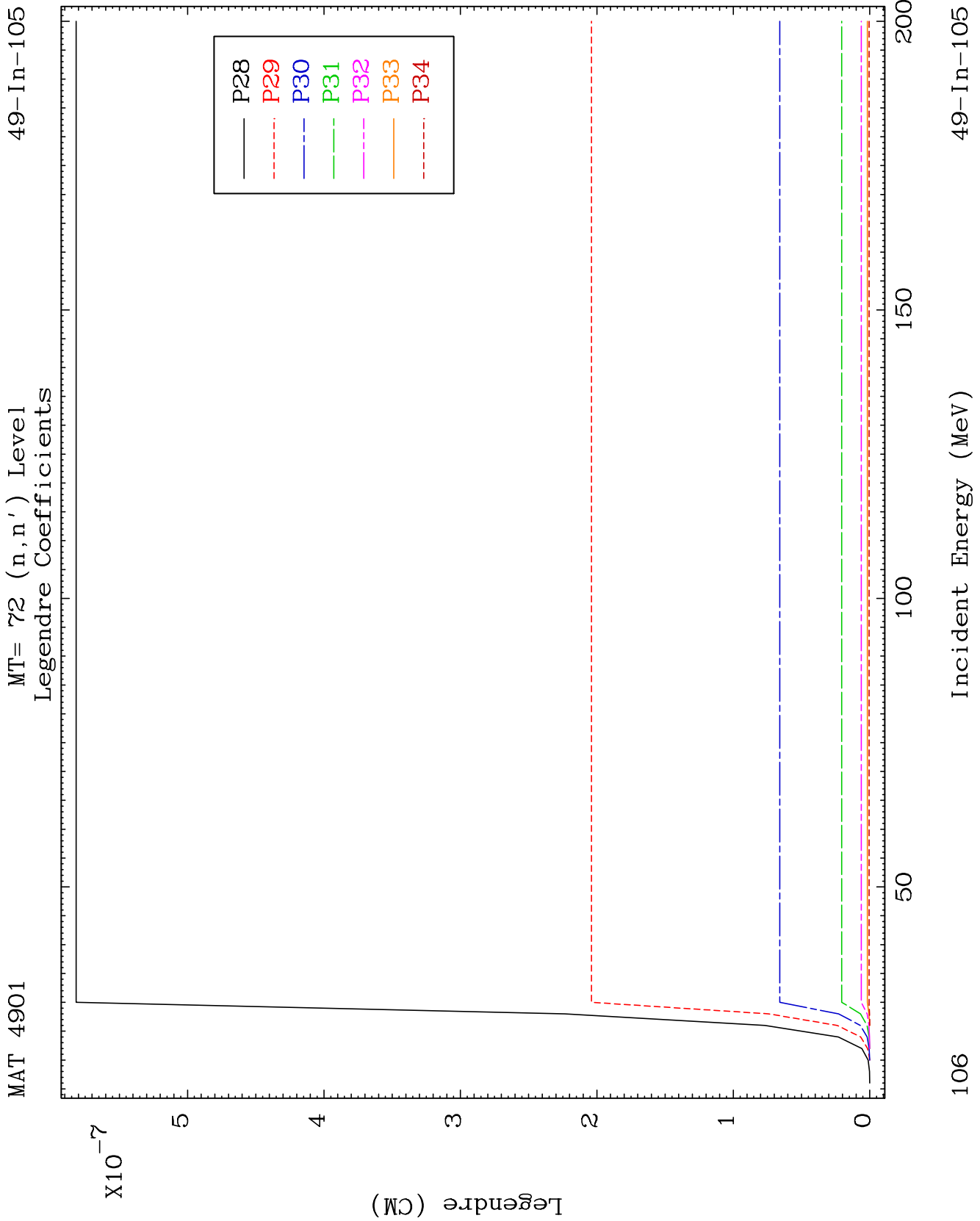
MAT 4901

MT= 72 (n, n') Level
Legendre Coefficients

49-In-105



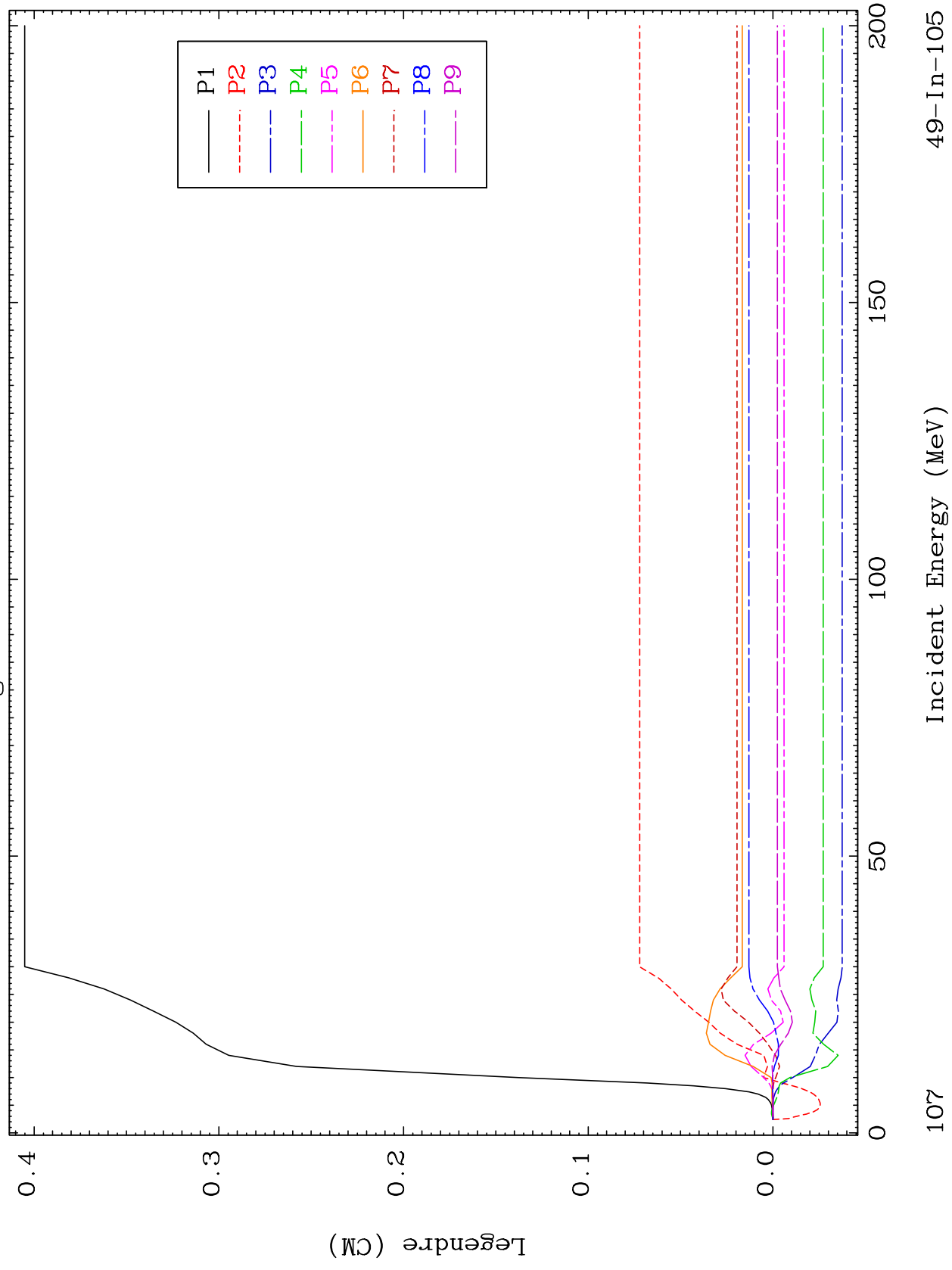


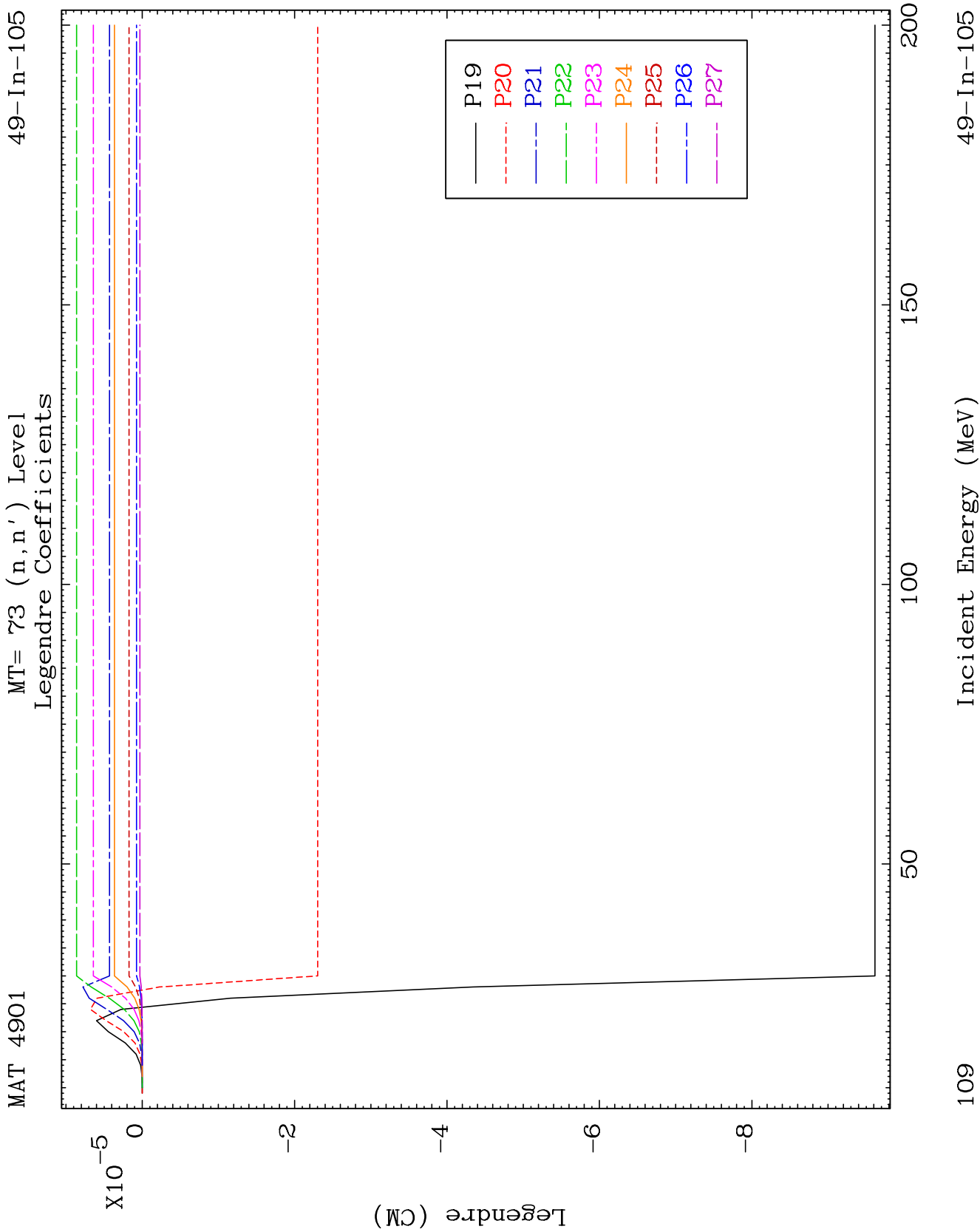


MAT 4901

MT= 73 (n, n') Level
Legendre Coefficients

49-In-105

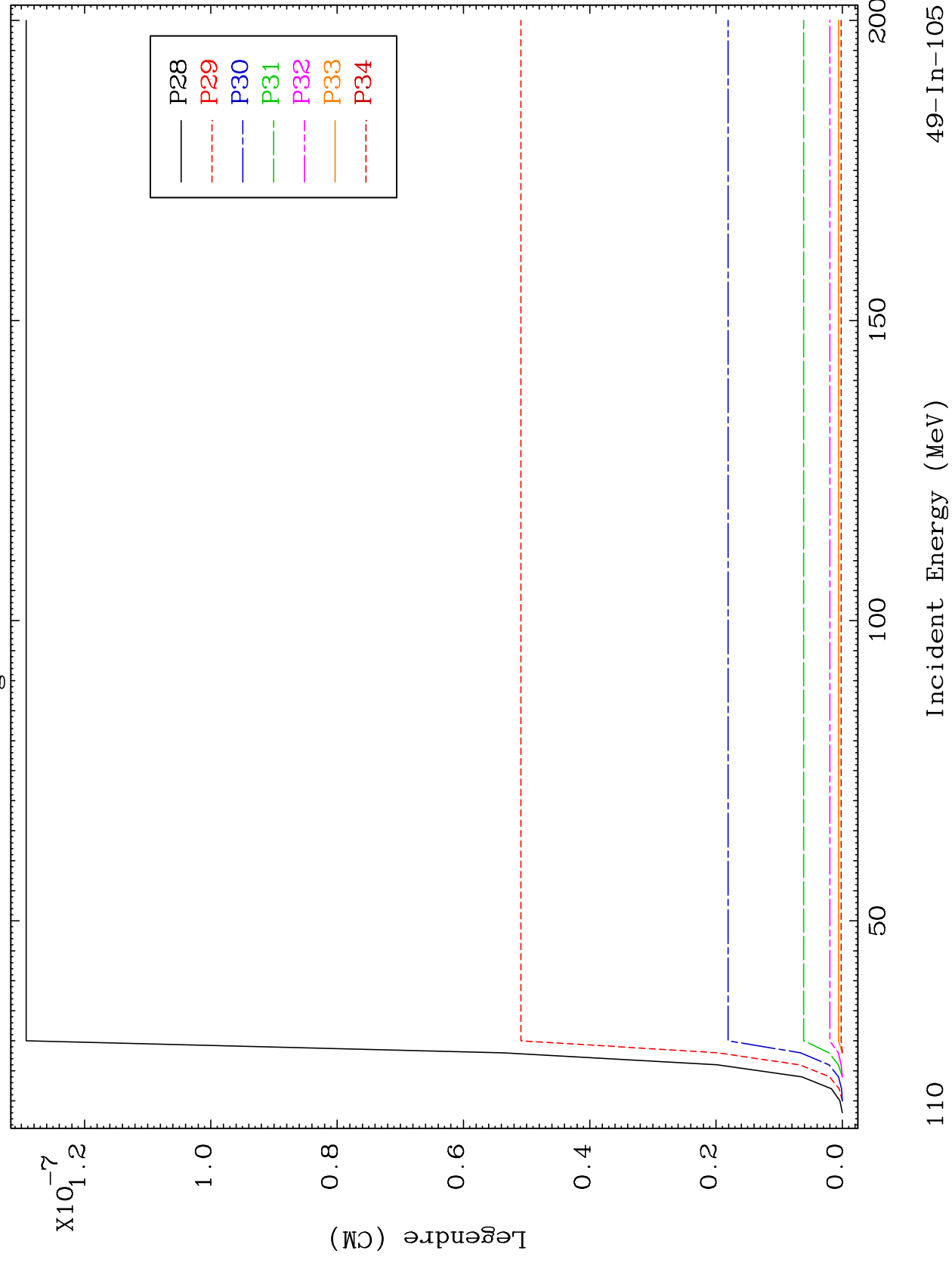


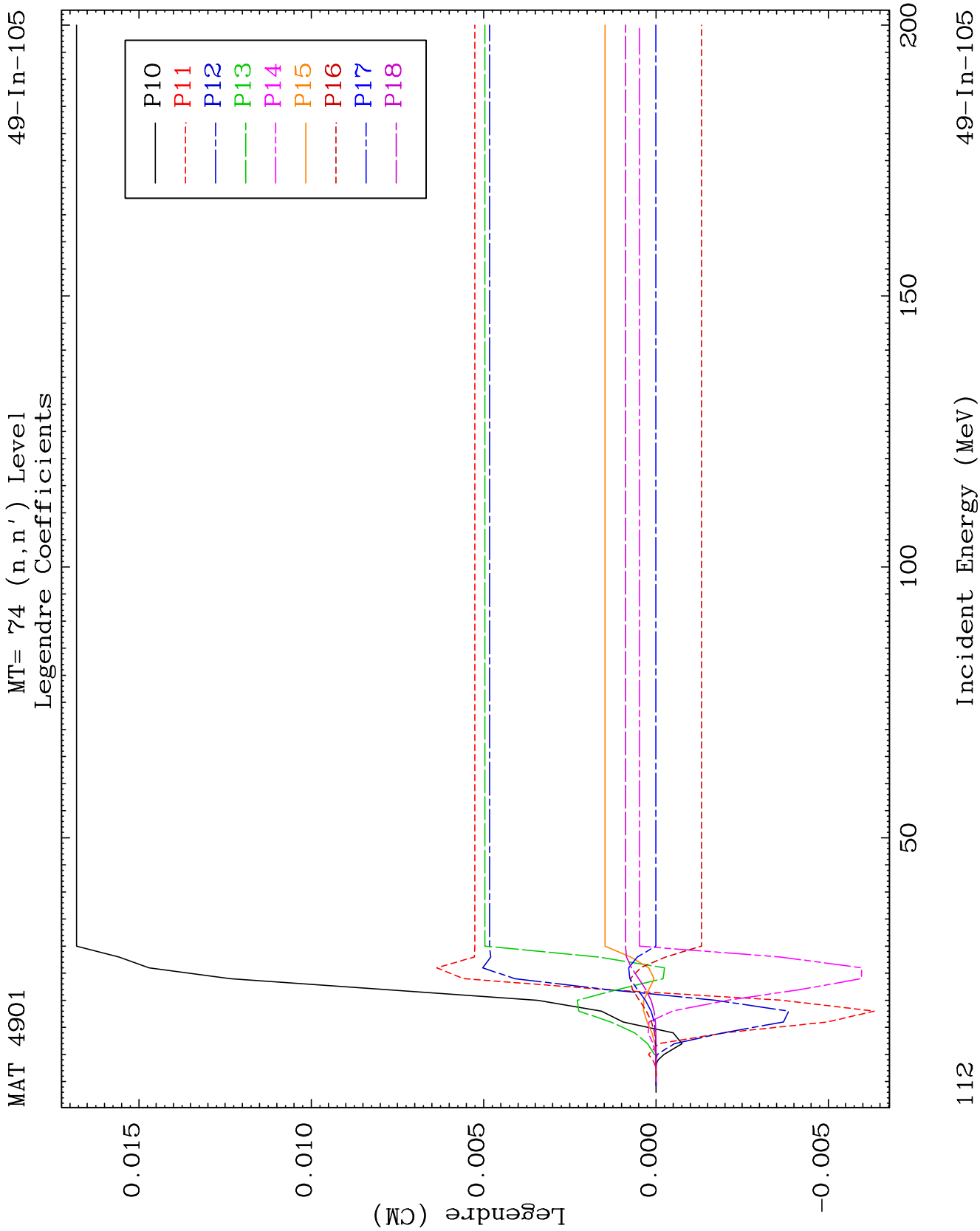


MAT 4901

MT= 73 (n, n') Level
Legendre Coefficients

49-In-105

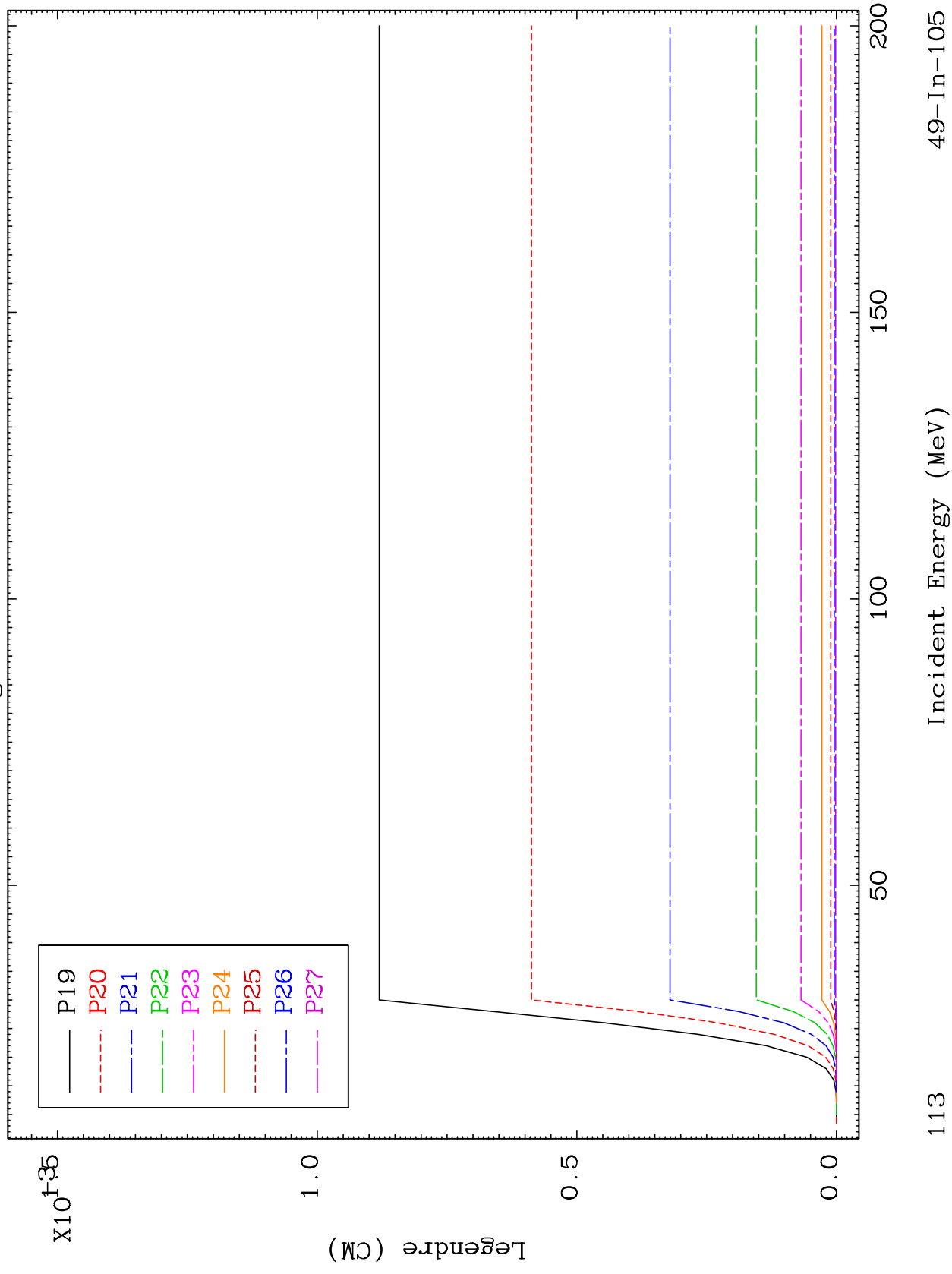


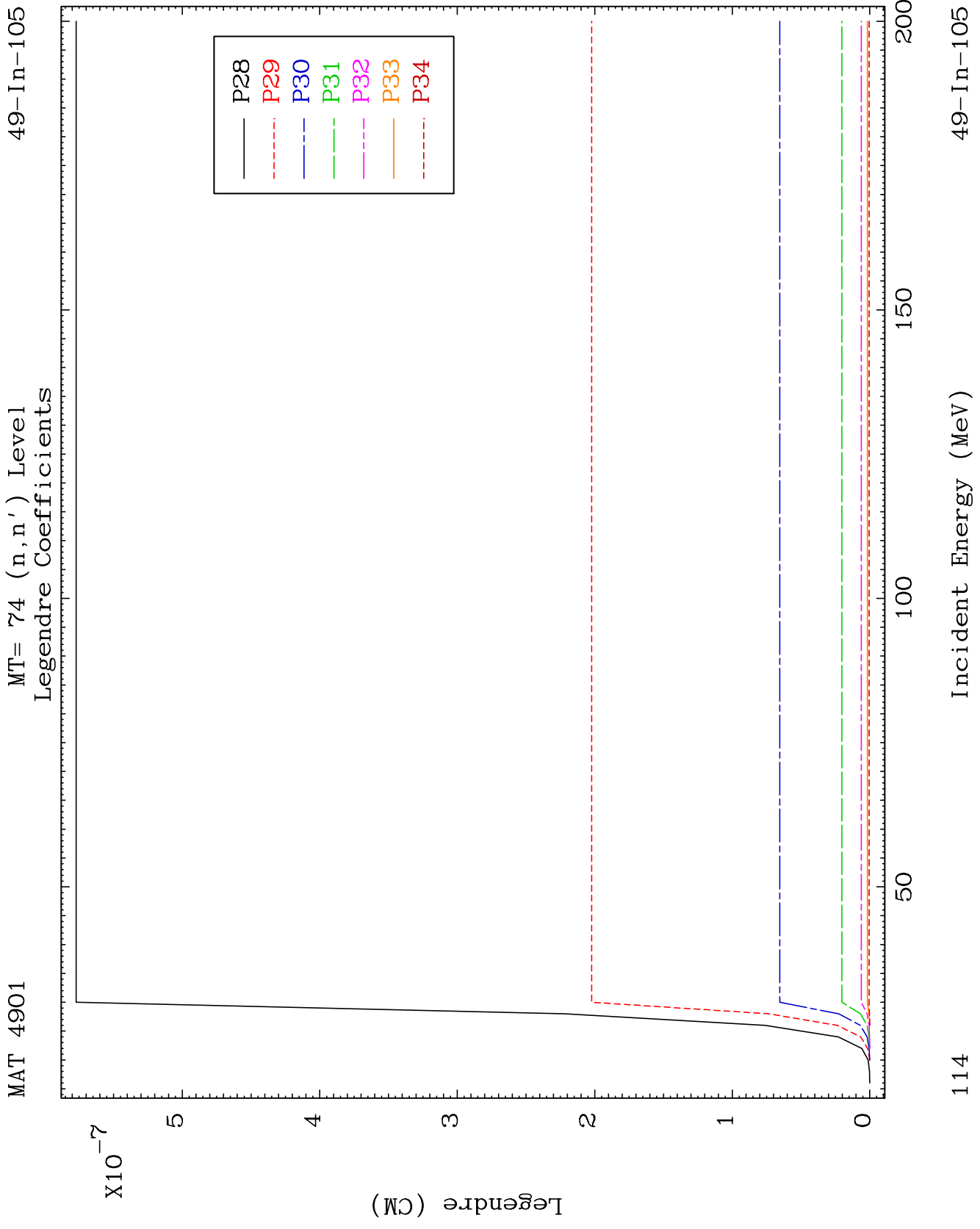


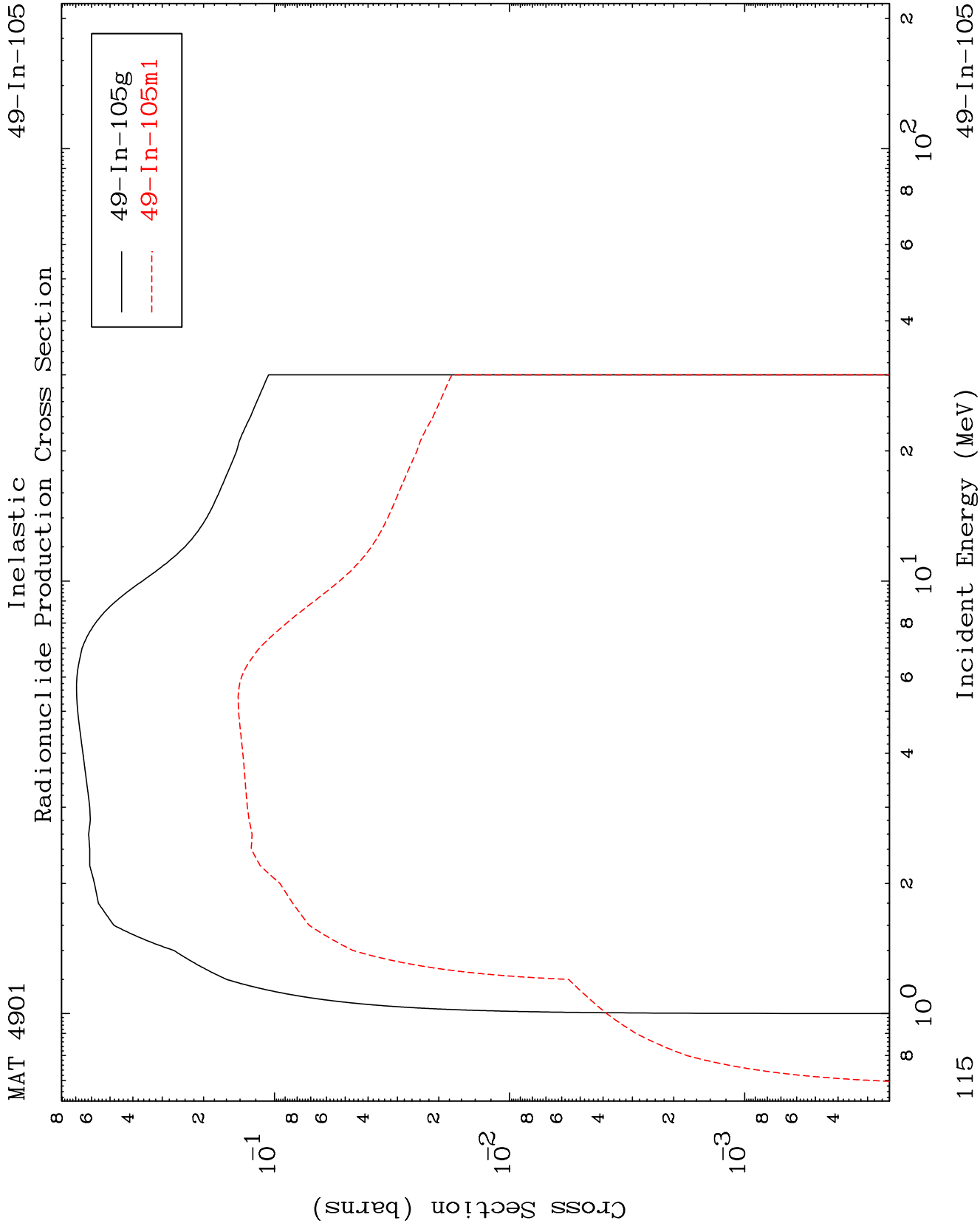
MAT 4901

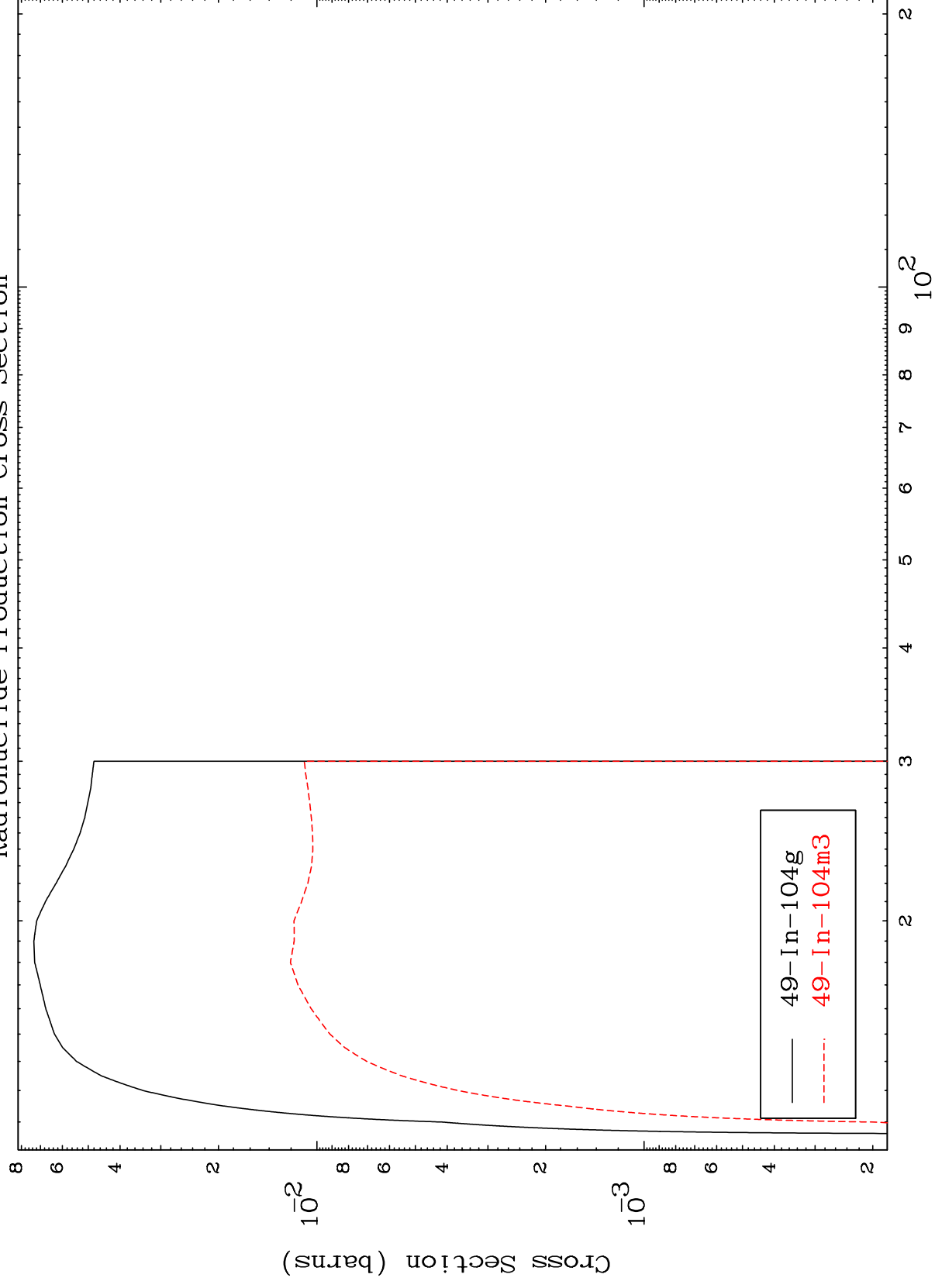
MT= 74 (n, n') Level
Legendre Coefficients

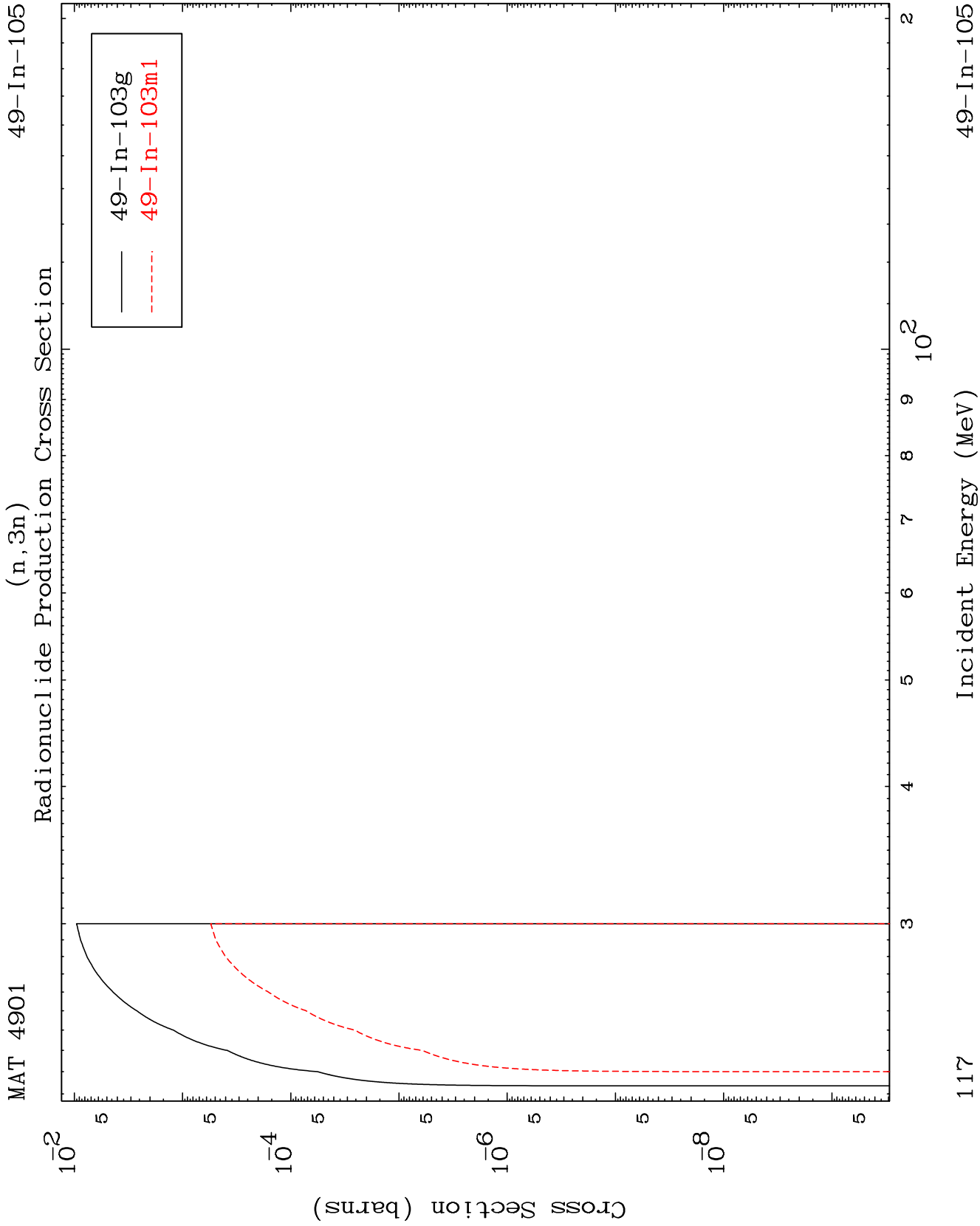
49-In-105







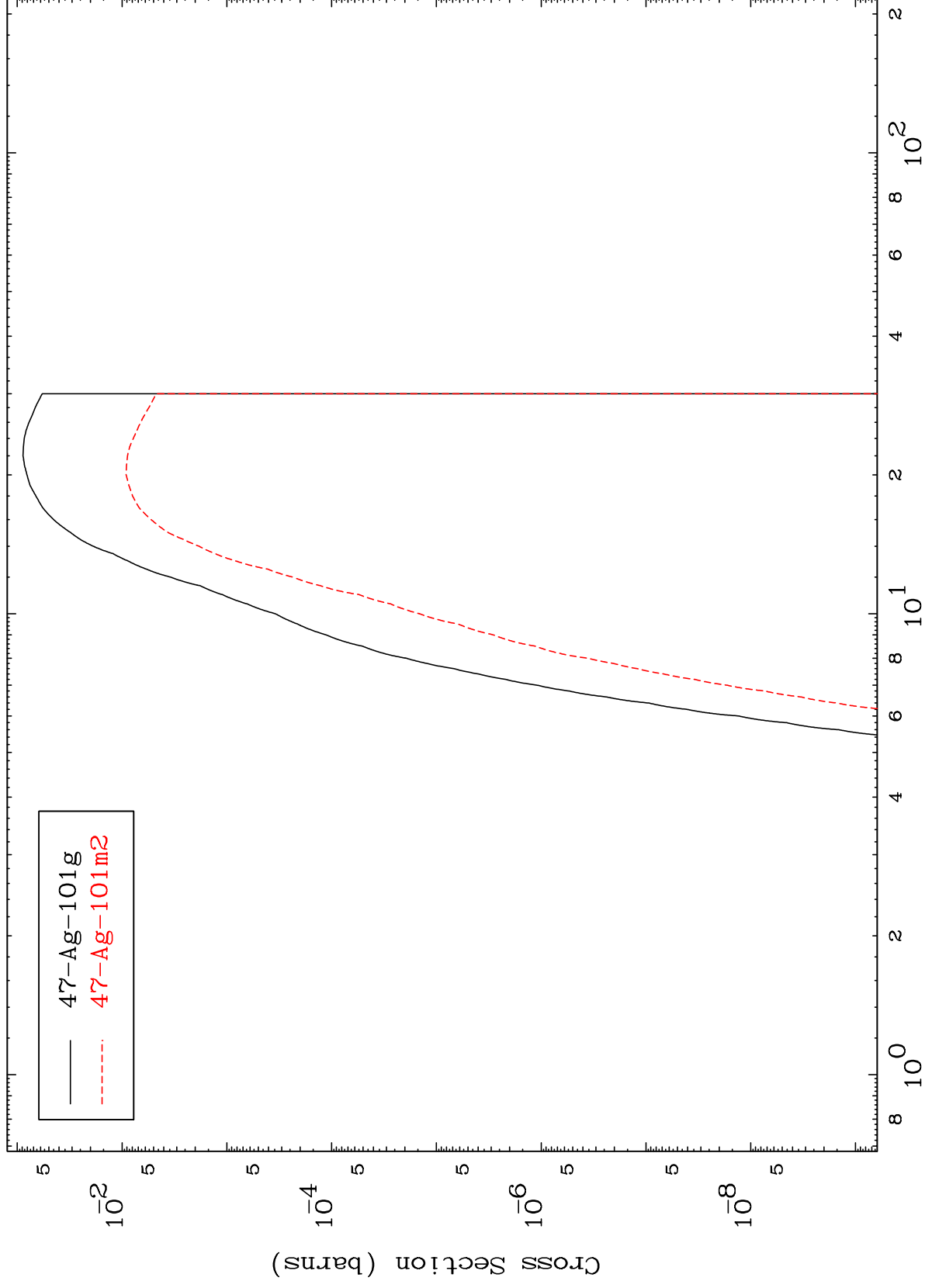
(n,2n)
Radionuclide Production Cross Section



MAT 4901

49-In-105

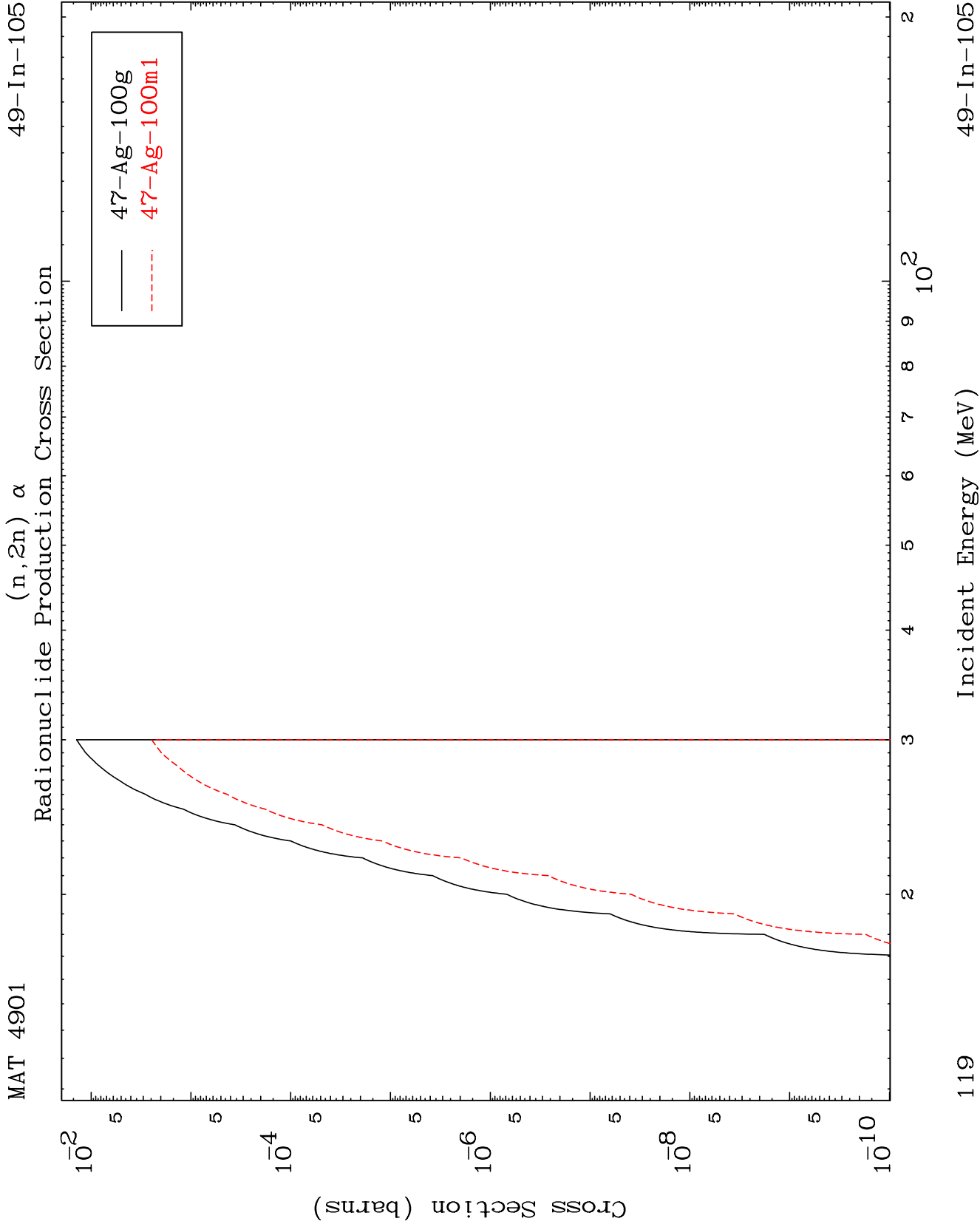
Radionuclide Production Cross Section
(n,n') α



118

Incident Energy (MeV)

49-In-105

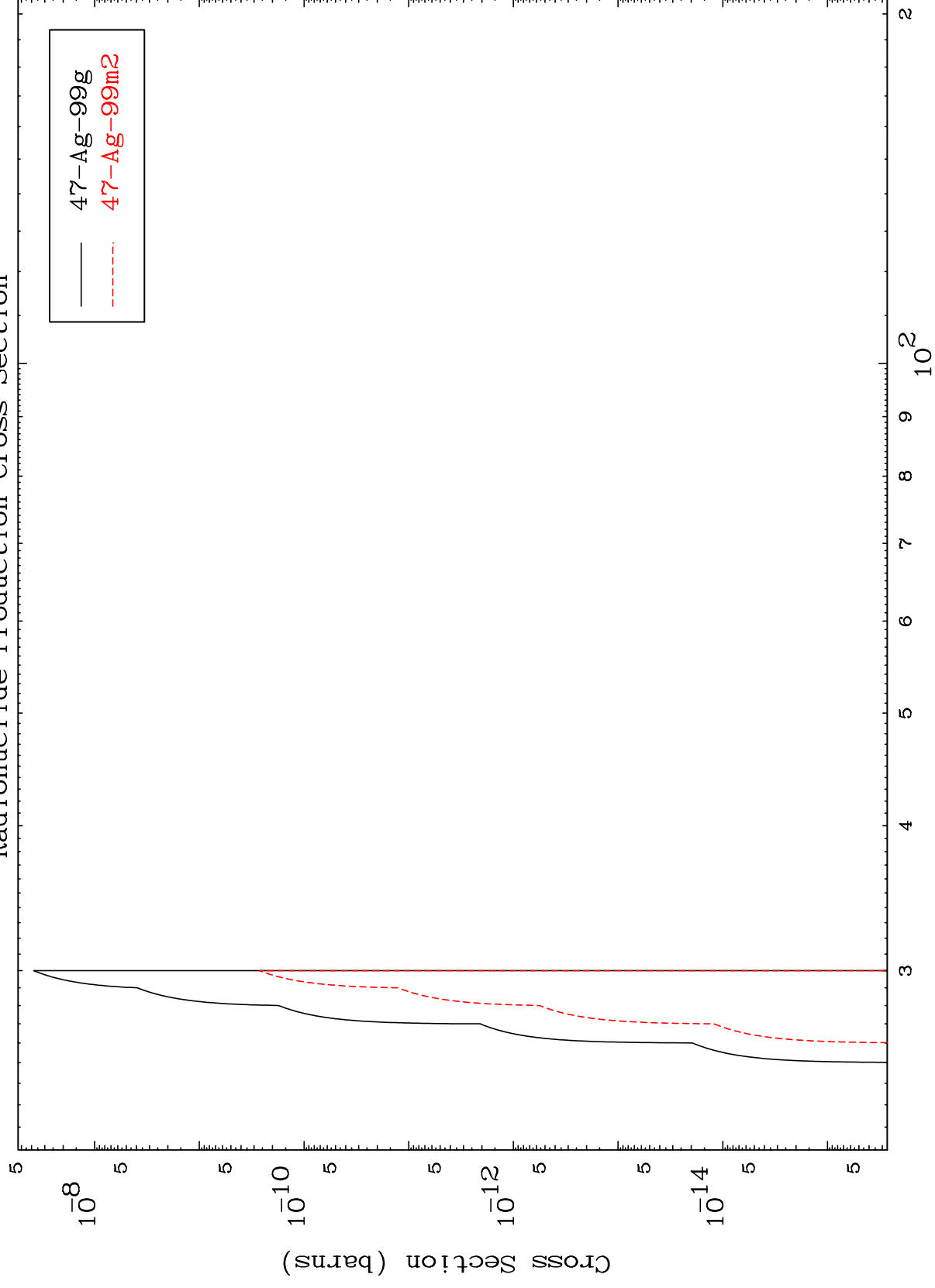


MAT 4901

$$(n, 3n) \propto$$

49-In-105

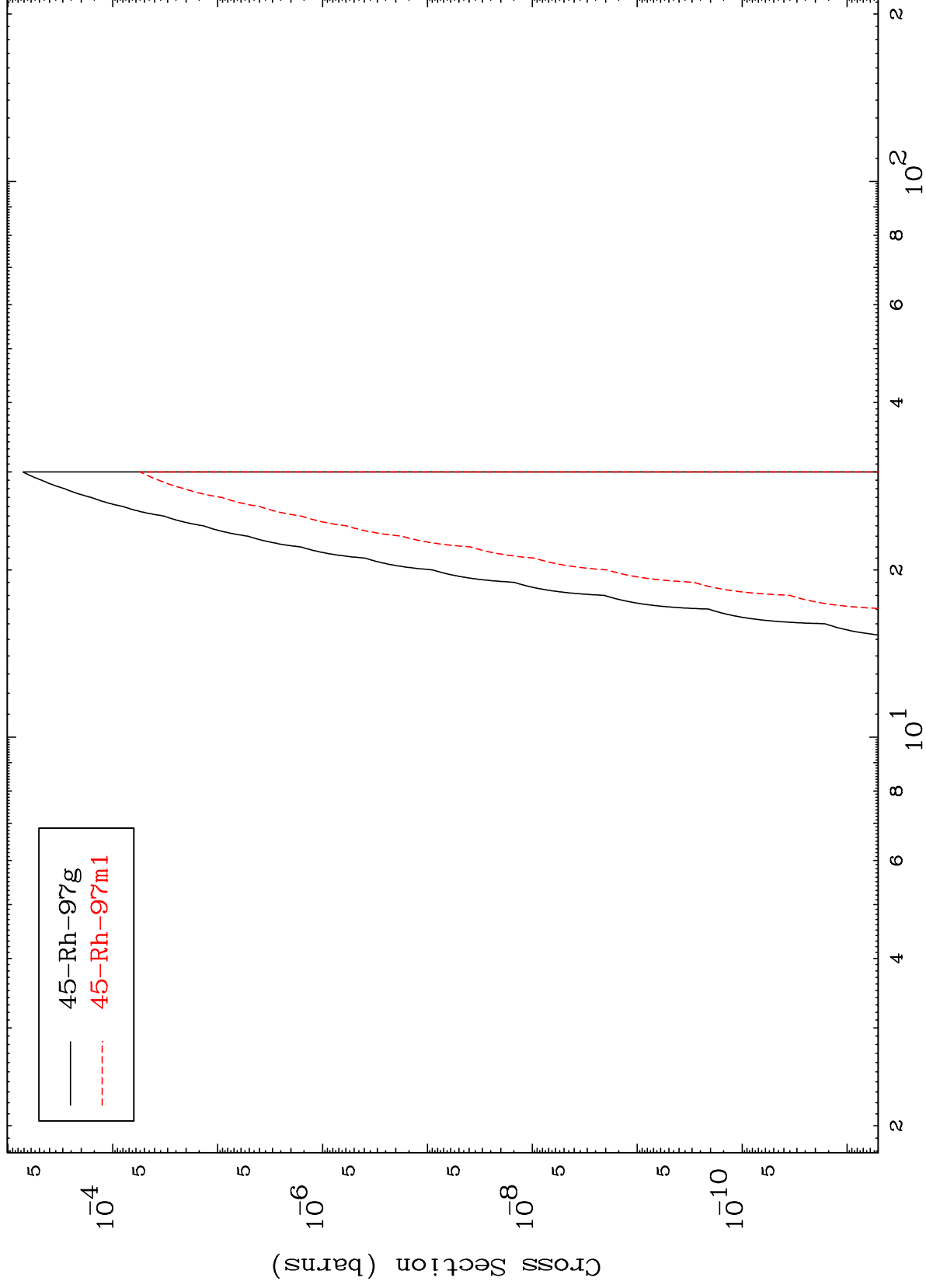
Radionuclide Production Cross Section



MAT 4901

49-In-105

(n,n') 2 α
Radionuclide Production Cross Section



121

Incident Energy (MeV)

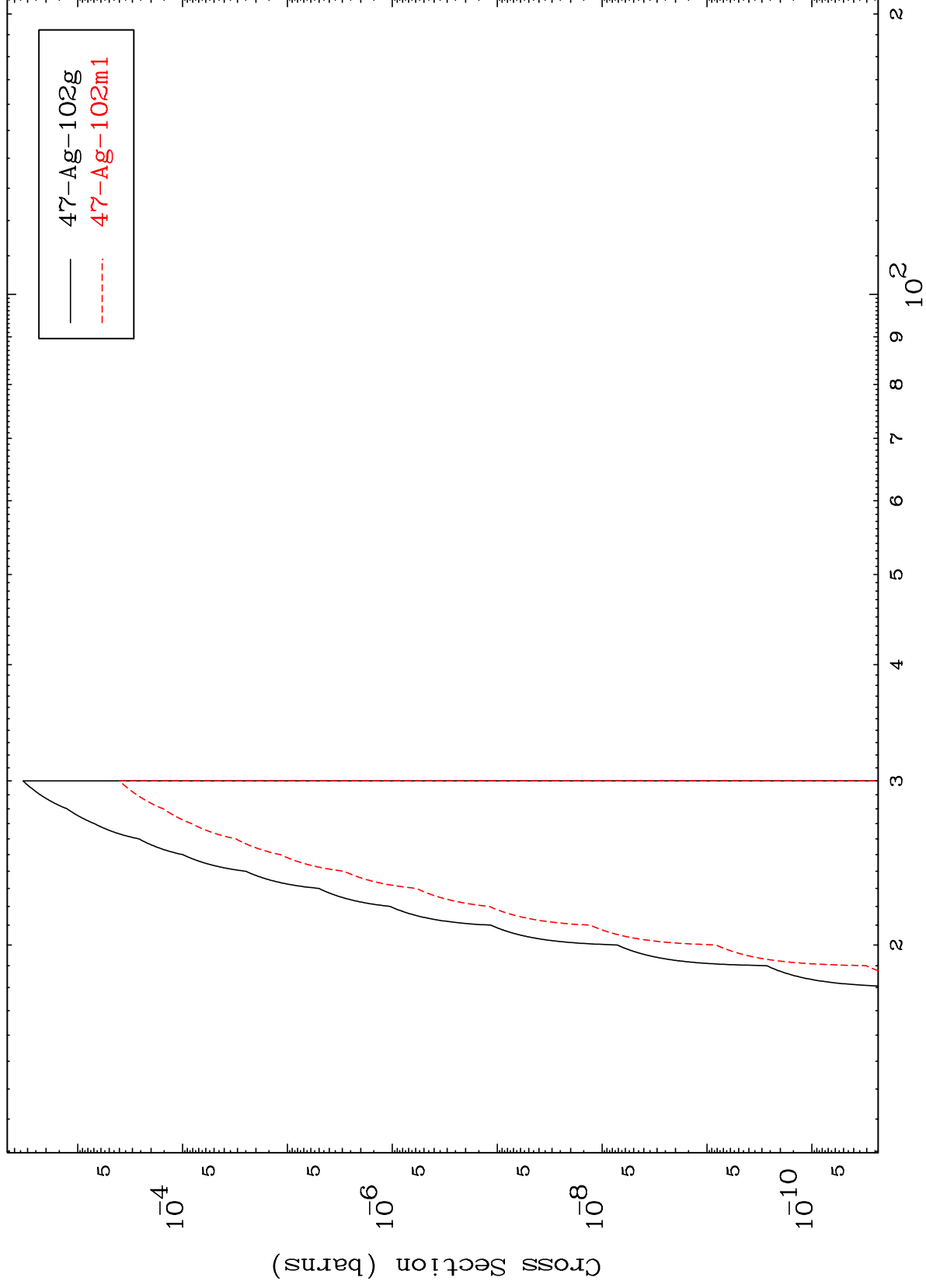
49-In-105

MAT 4901

(n,n') He-3

49-In-105

Radionuclide Production Cross Section



122

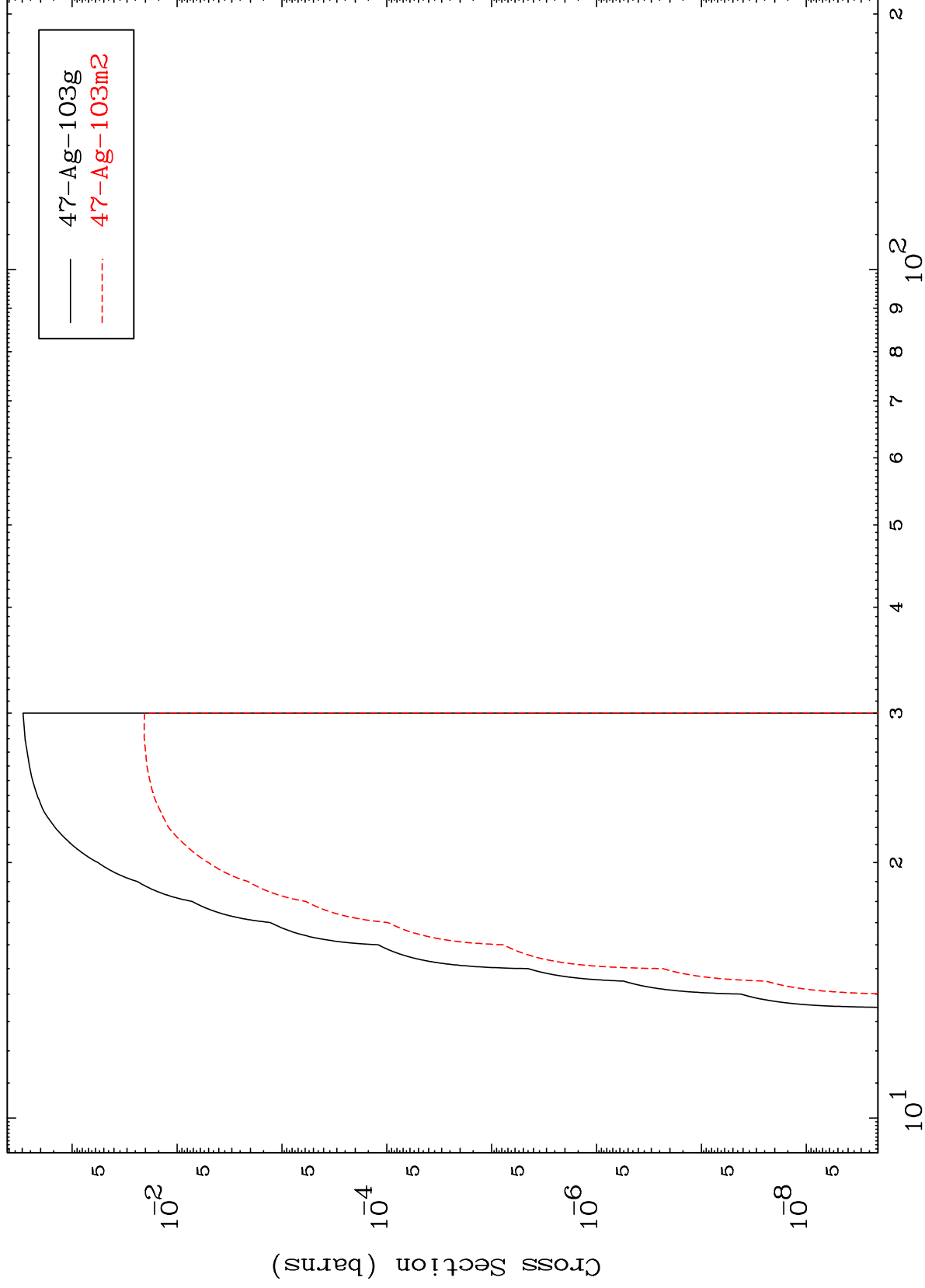
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

(n,2n) p
Radionuclide Production Cross Section



49-In-105

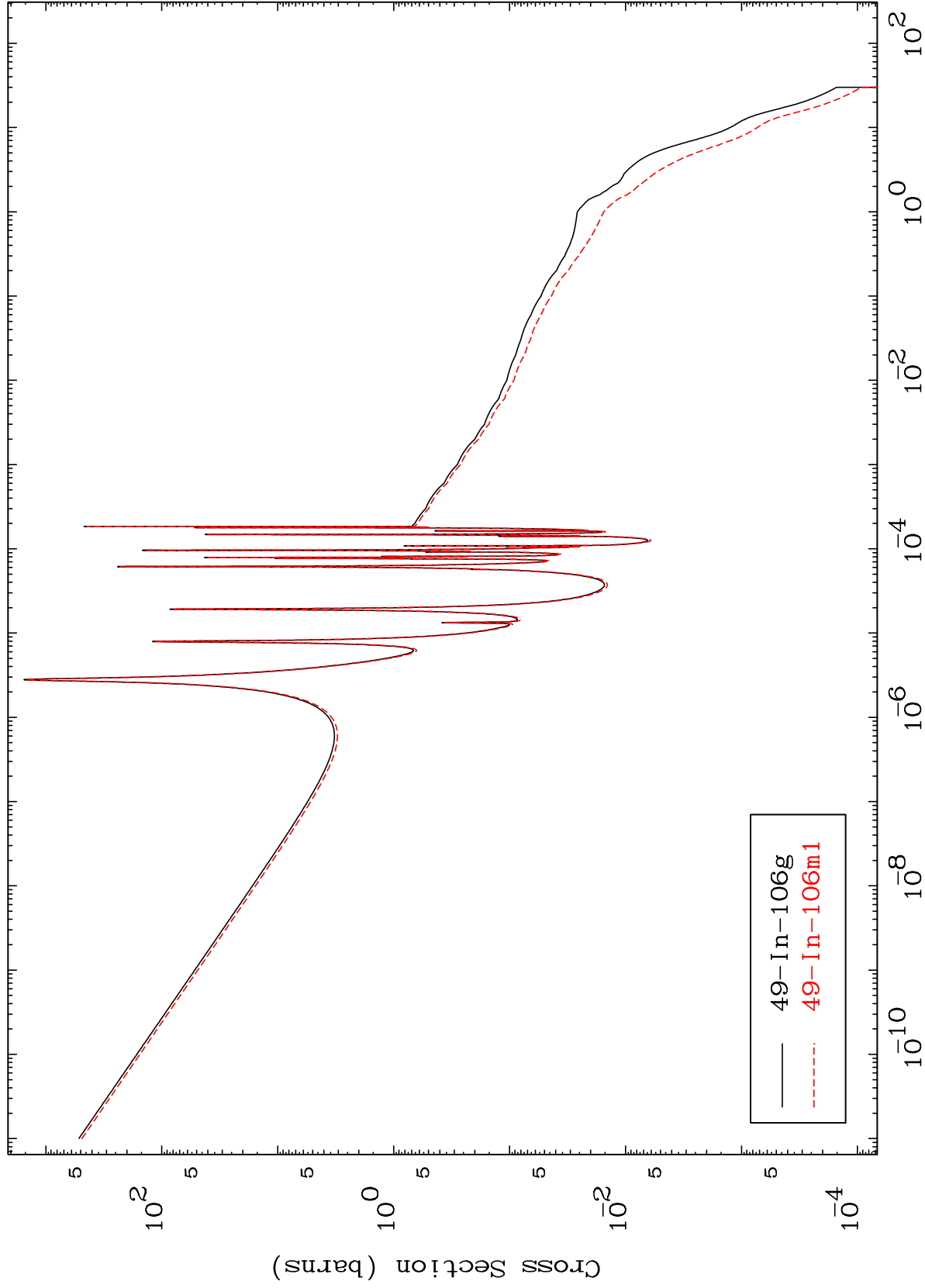
Incident Energy (MeV)

123

MAT 4901

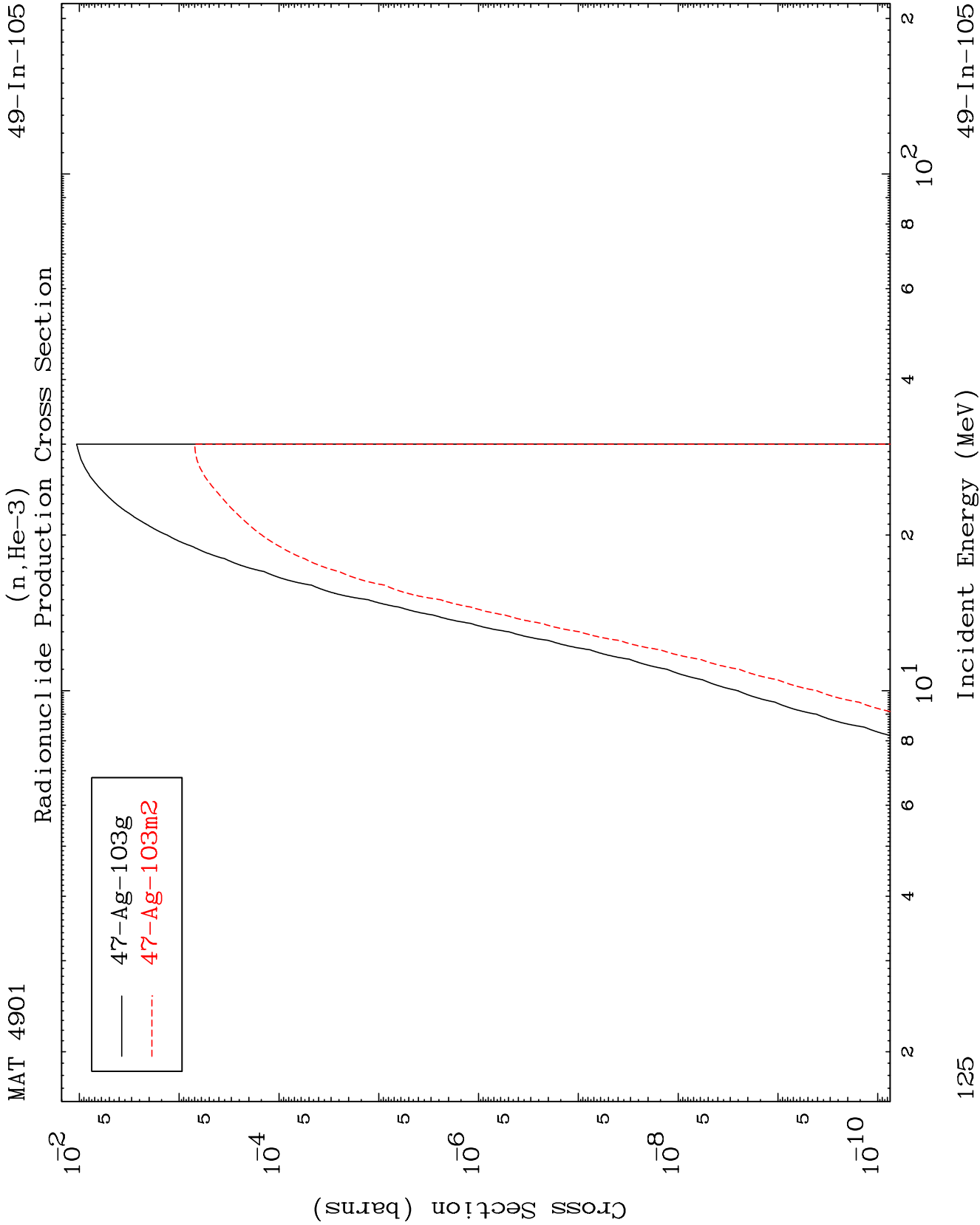
49-In-105

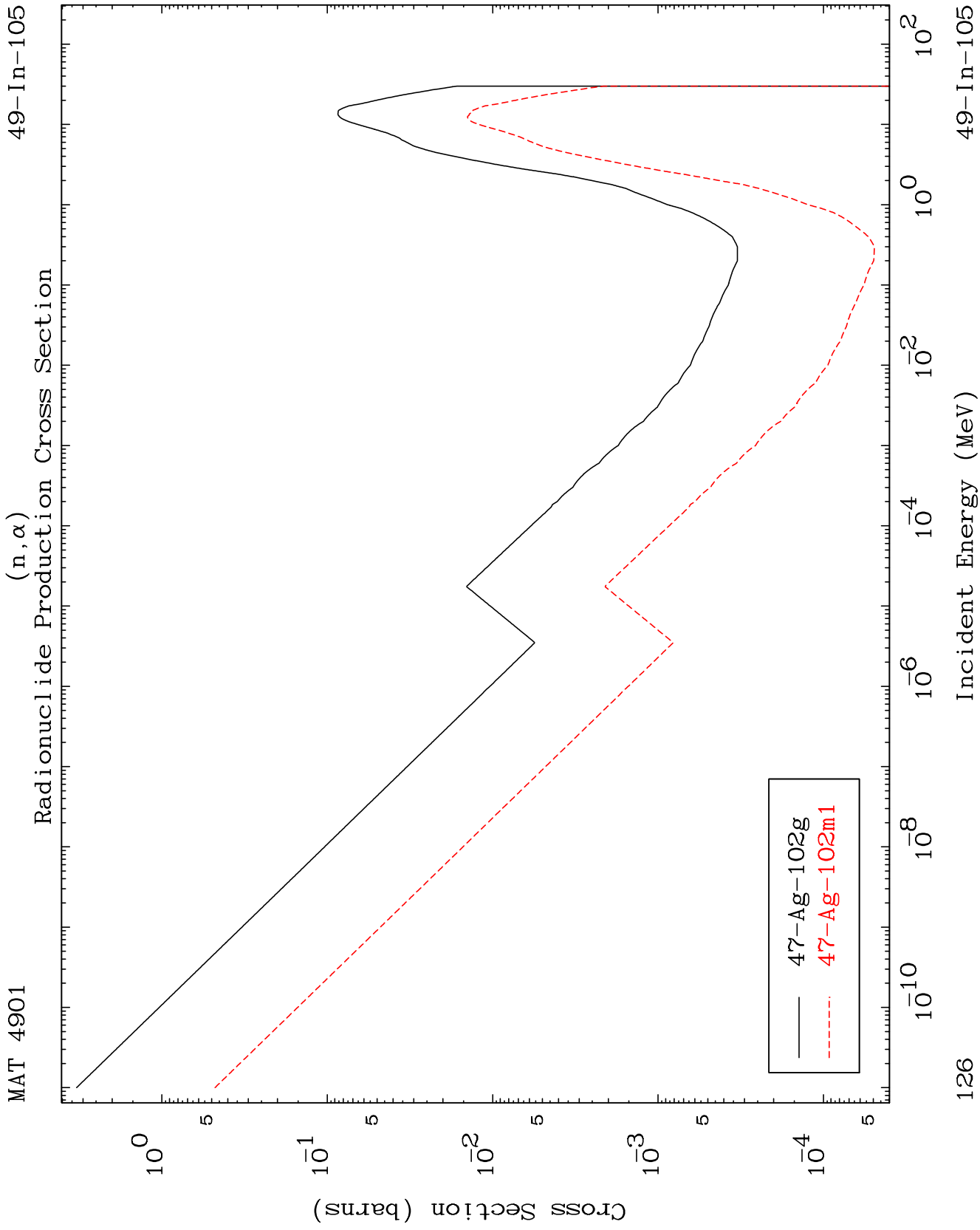
(n, γ)
Radionuclide Production Cross Section



124

49-In-105

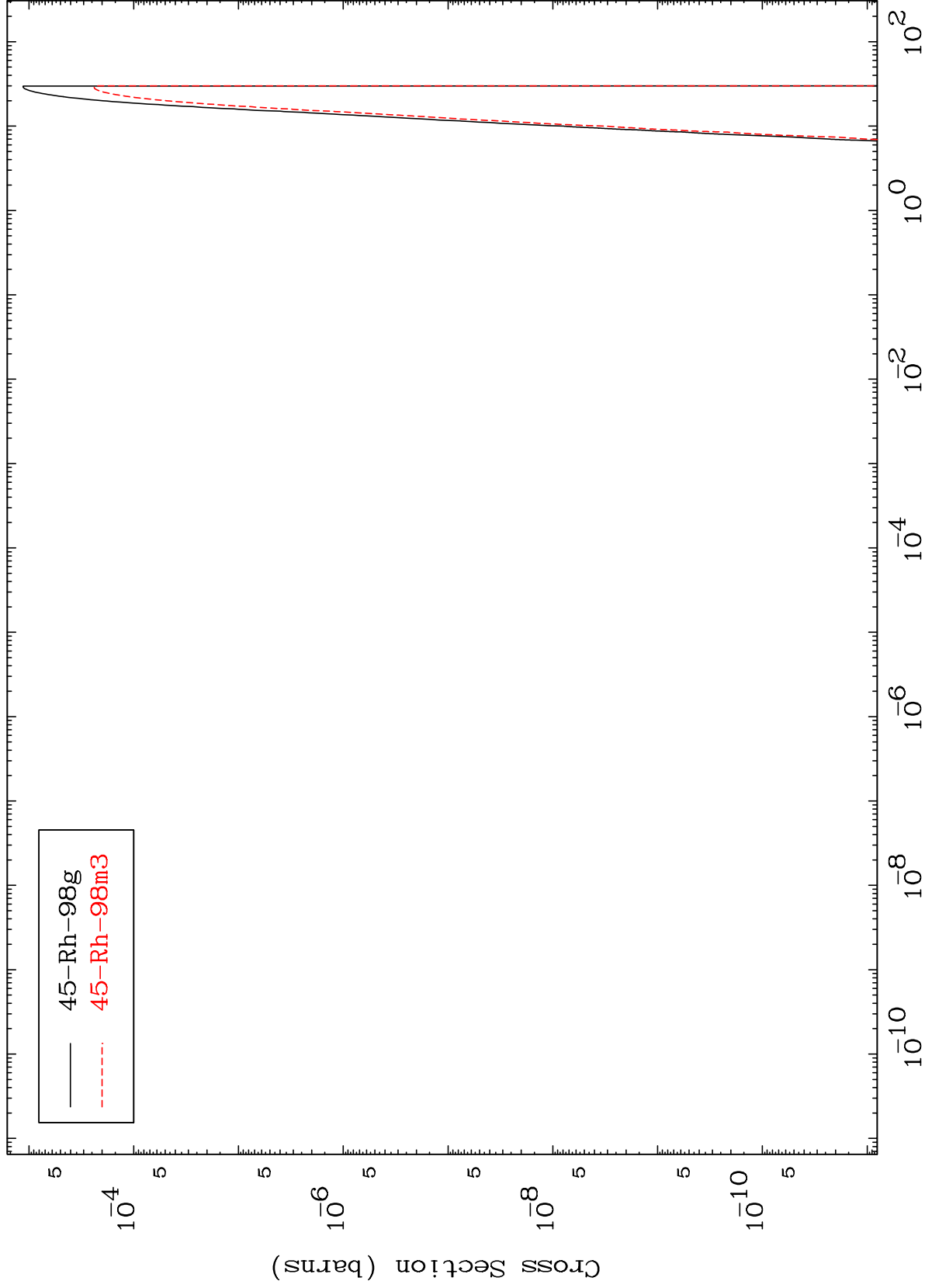




MAT 4901

49-In-105

(n,2 α)
Radionuclide Production Cross Section



127

Incident Energy (MeV)

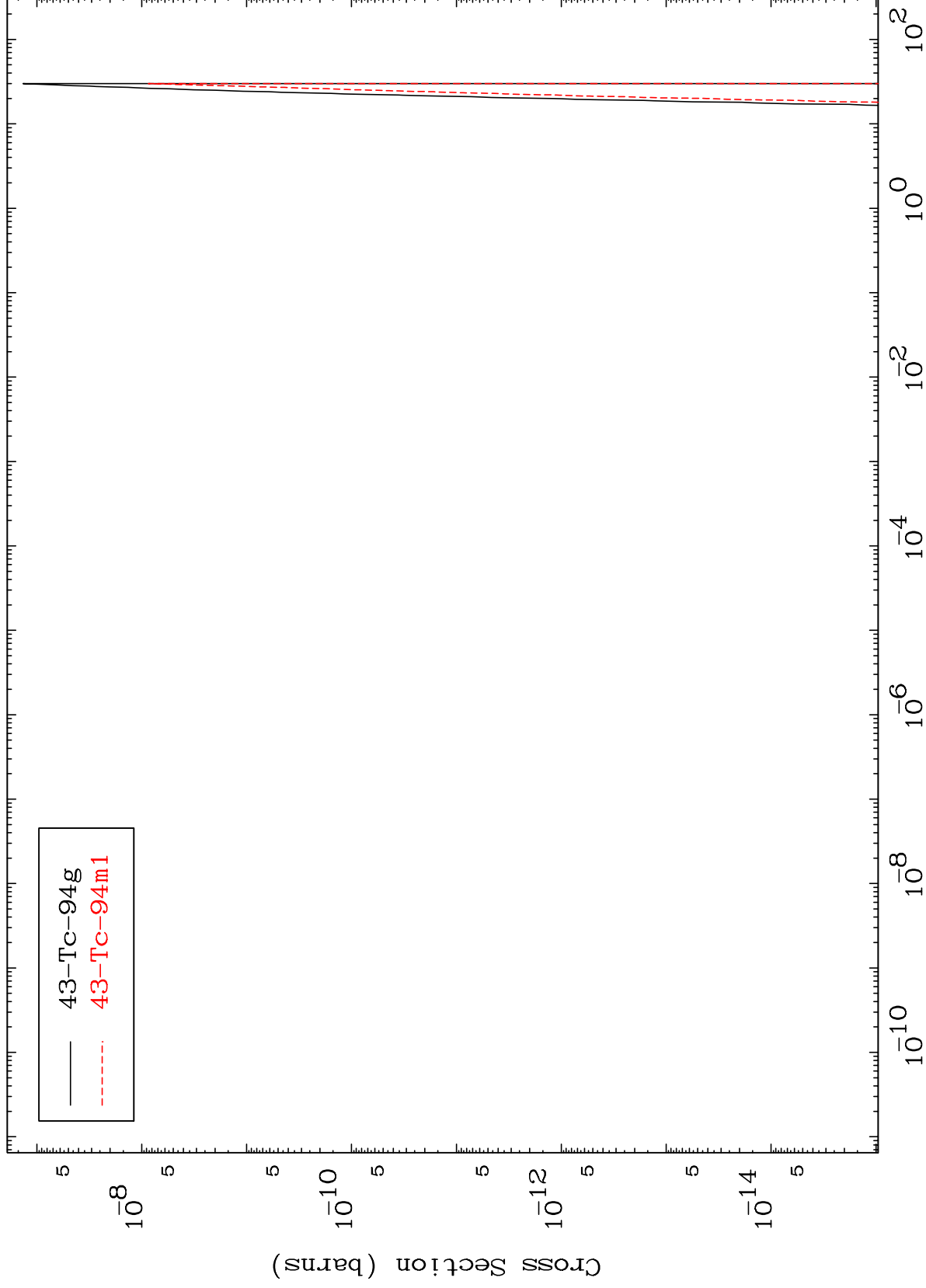
49-In-105

MAT 4901

(n,3 α)

49-In-105

Radionuclide Production Cross Section



128

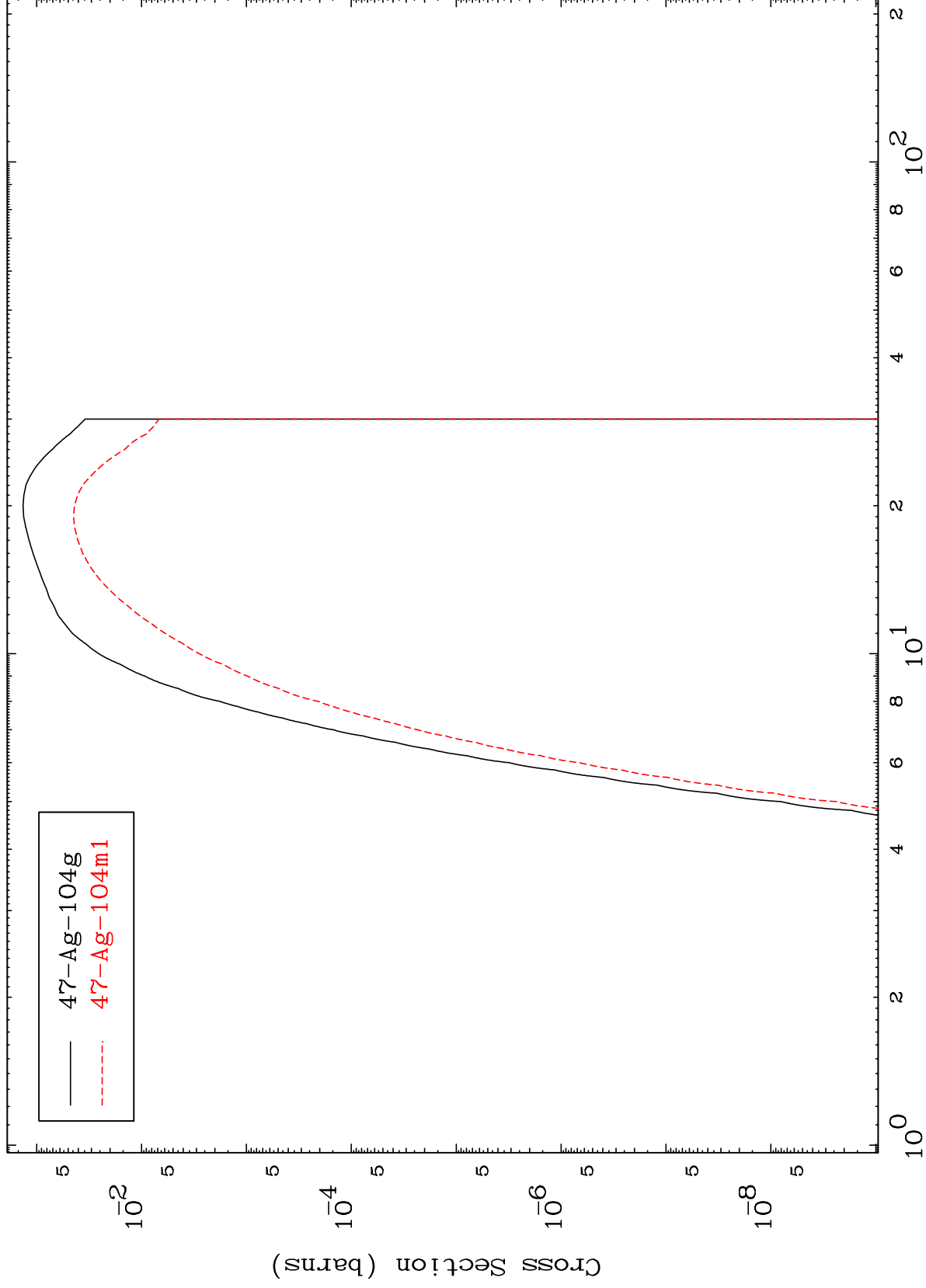
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

(n,2p)
Radionuclide Production Cross Section



Incident Energy (MeV)

49-In-105

129

