

Program EVALPLOT
(Version 2017-1)

by

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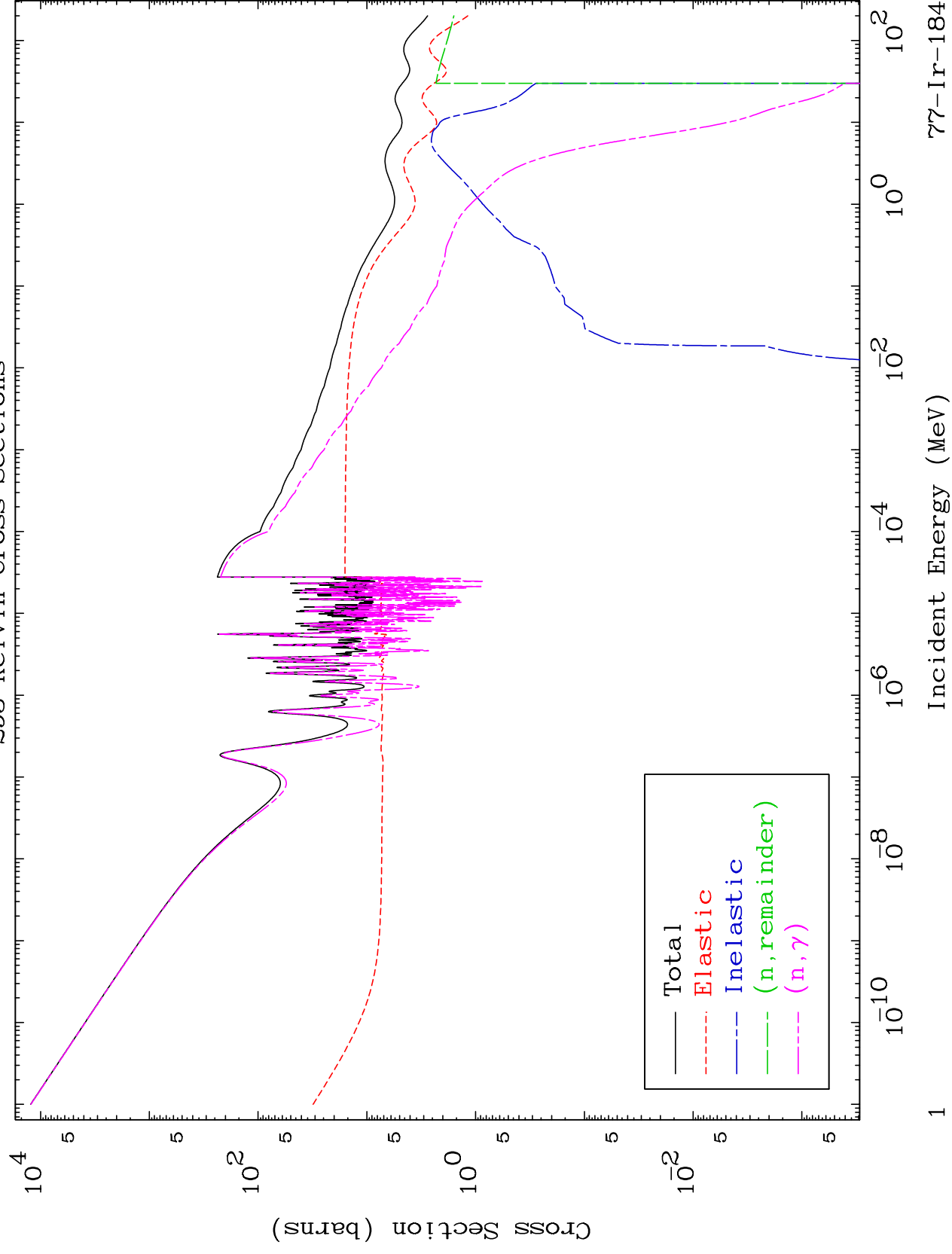
Press Mouse Button to Start

MAT 2704

Major

293 Kelvin Cross Sections

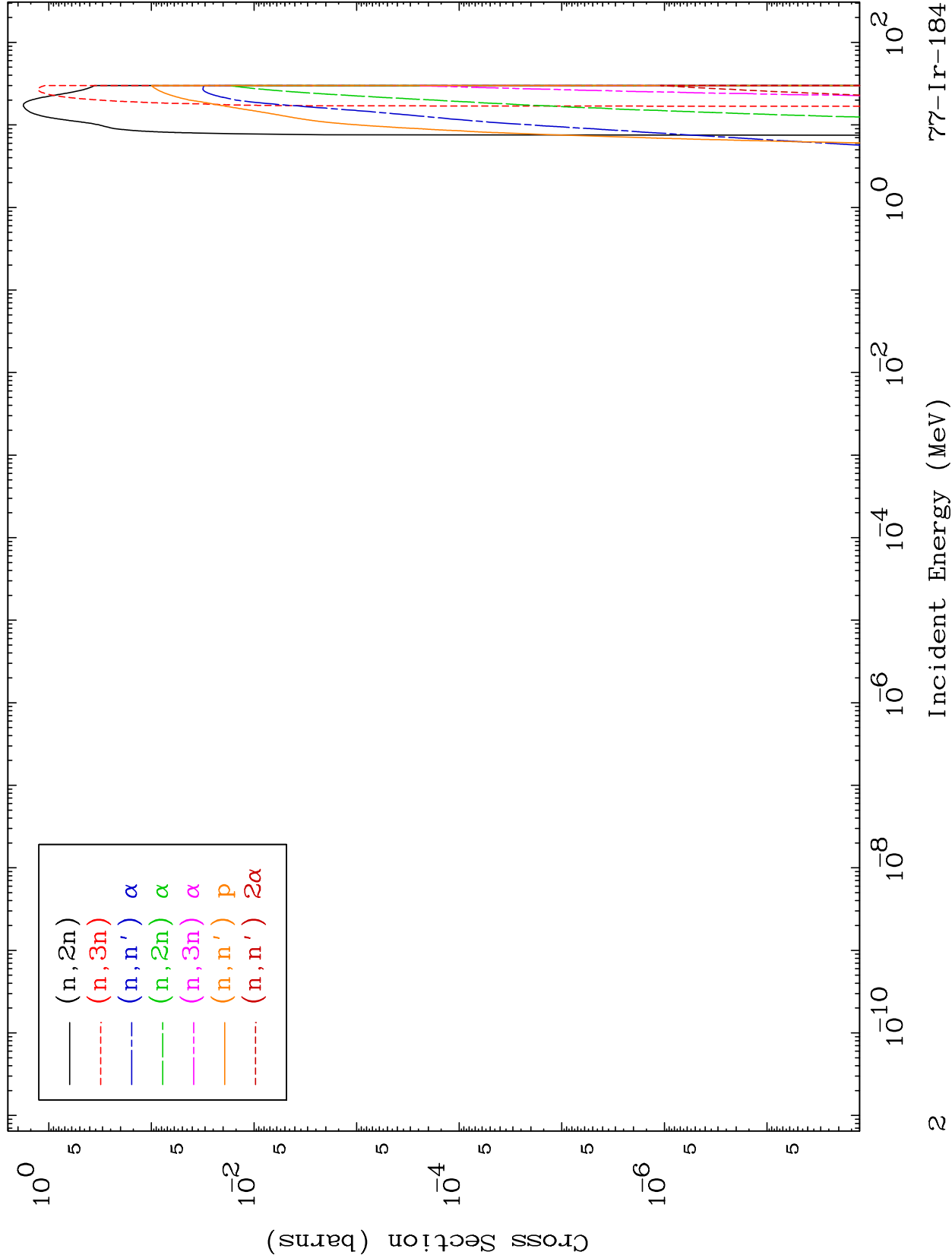
77-Ir-184



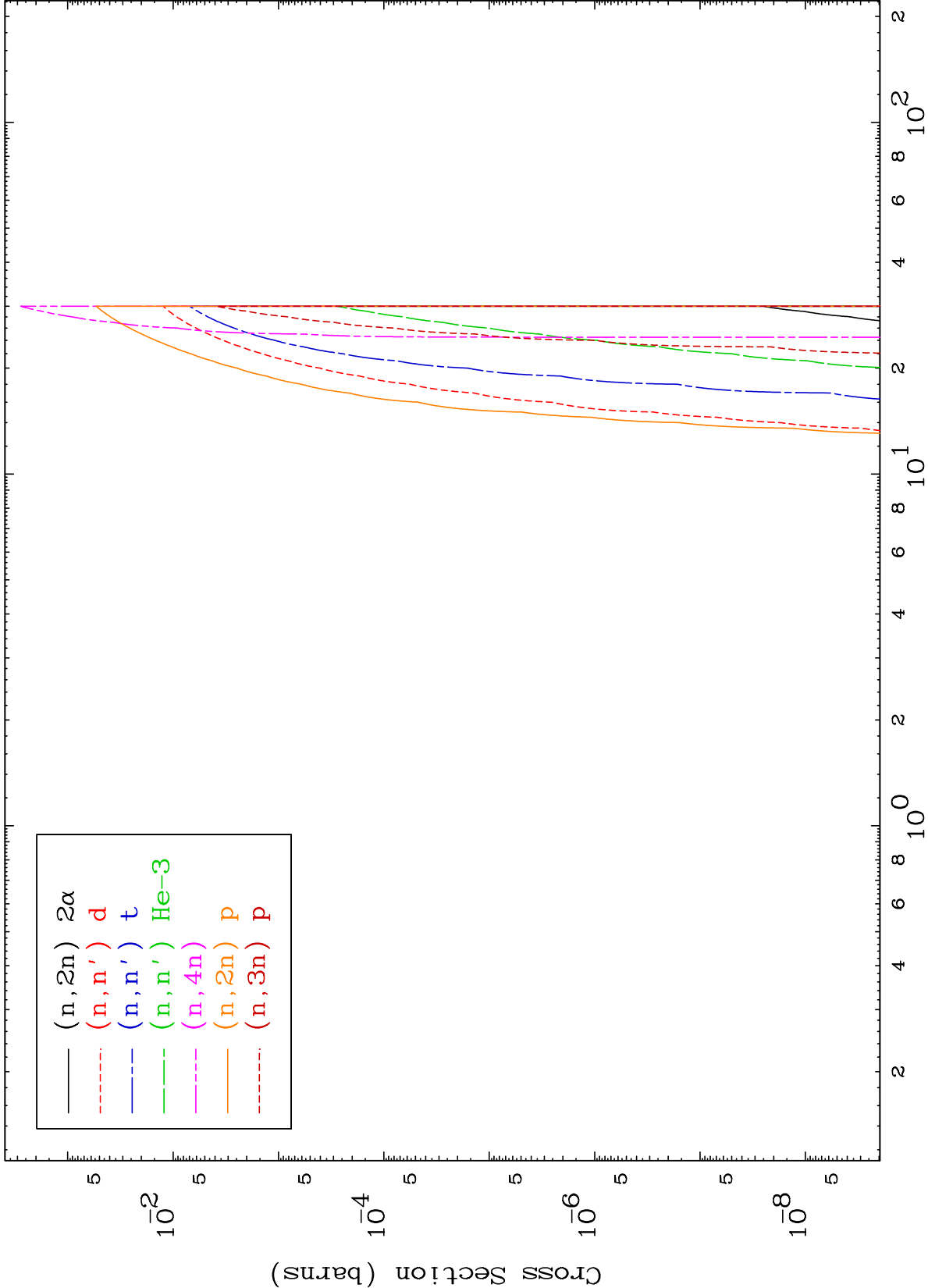
MAT 7704

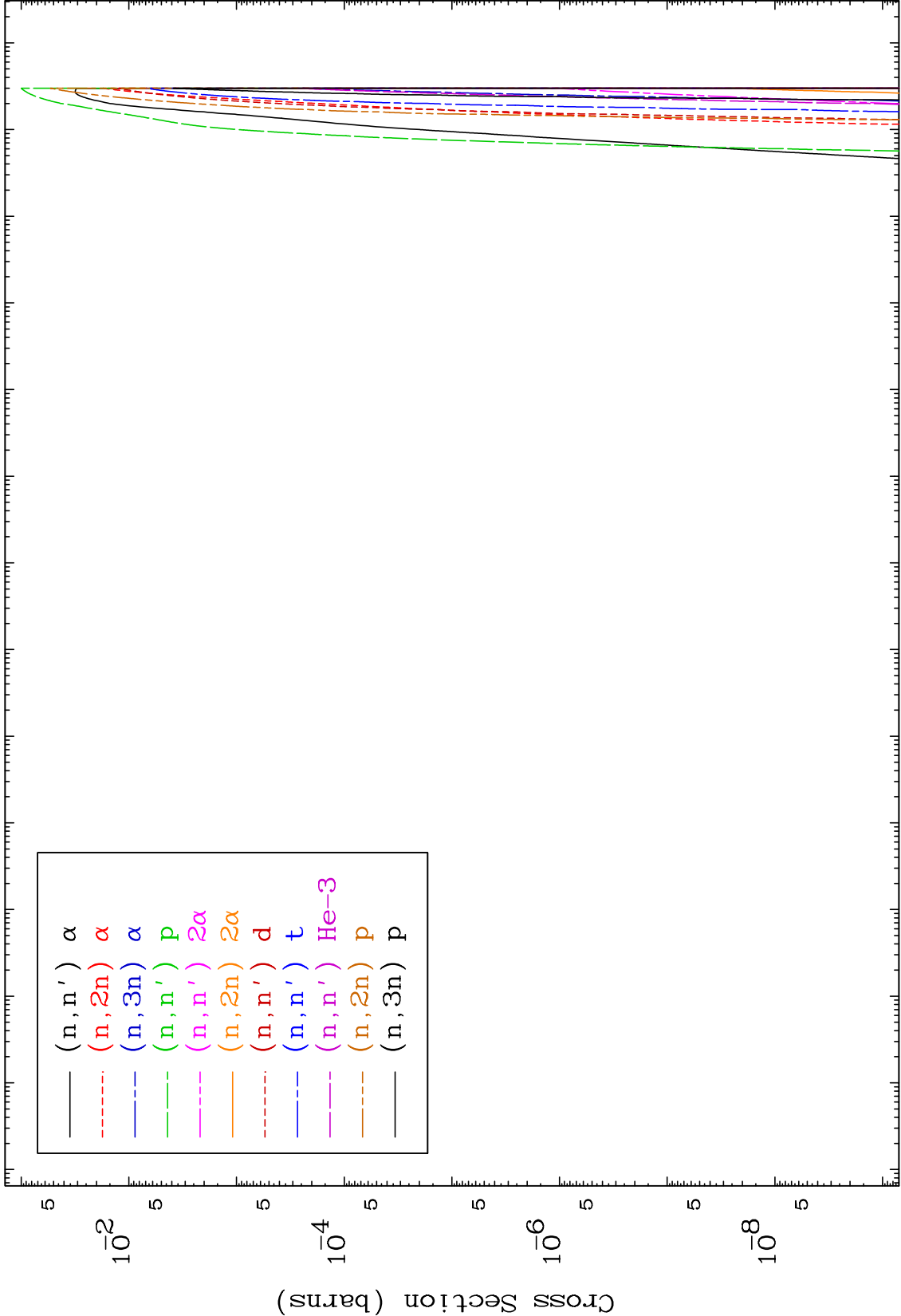
Neutron Production
293 Kelvin Cross Sections

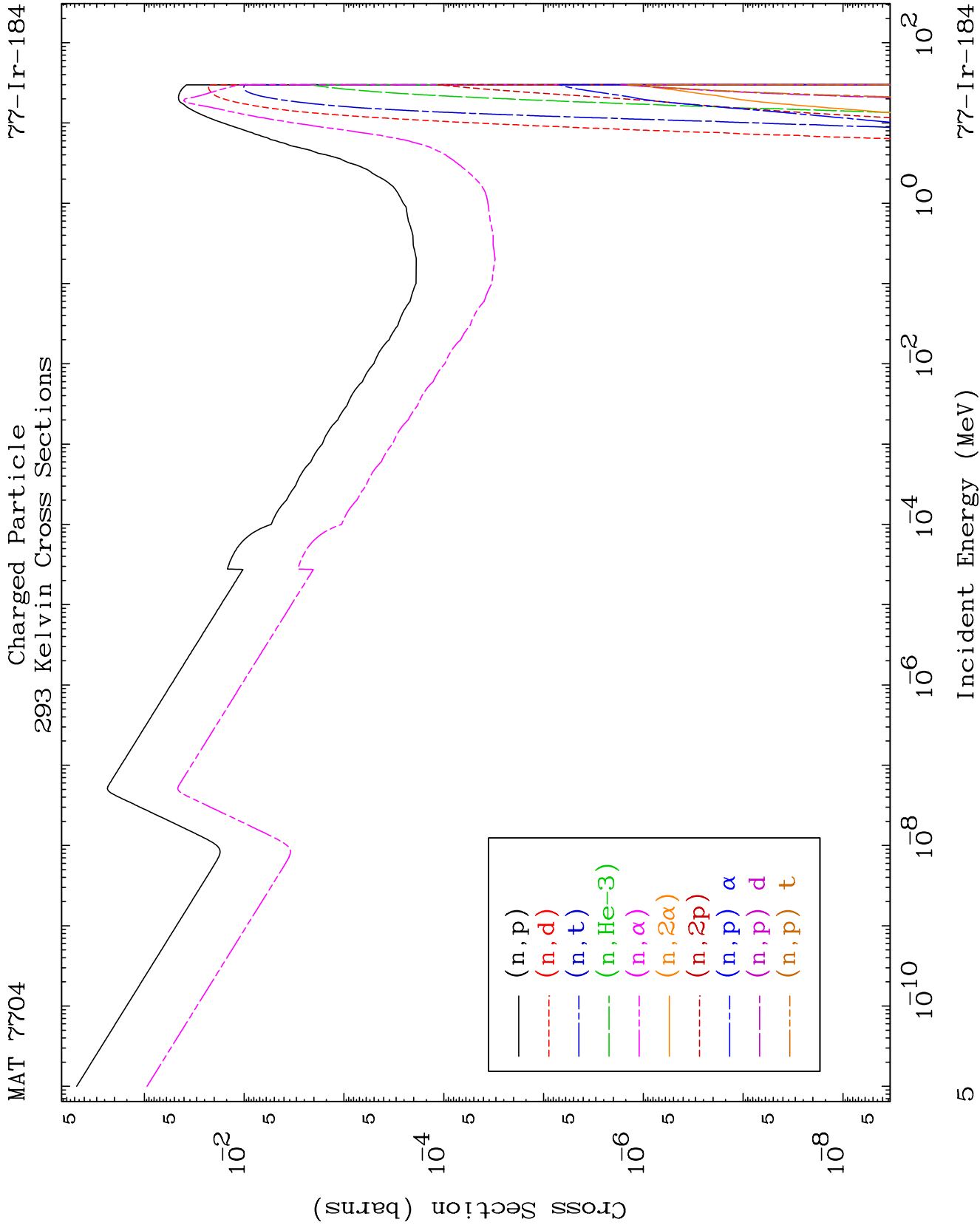
77-Ir-184

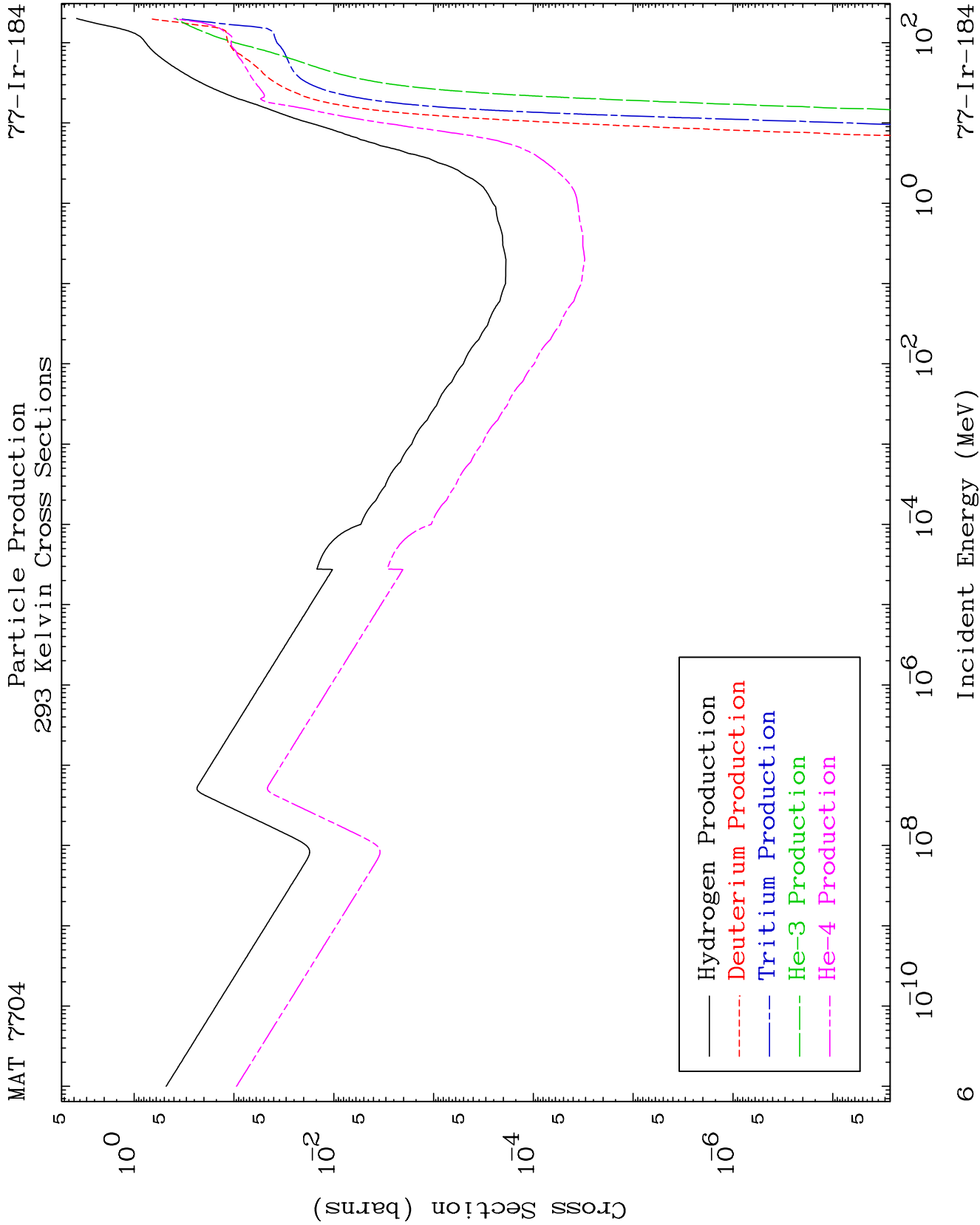


293 Kelvin Cross Sections

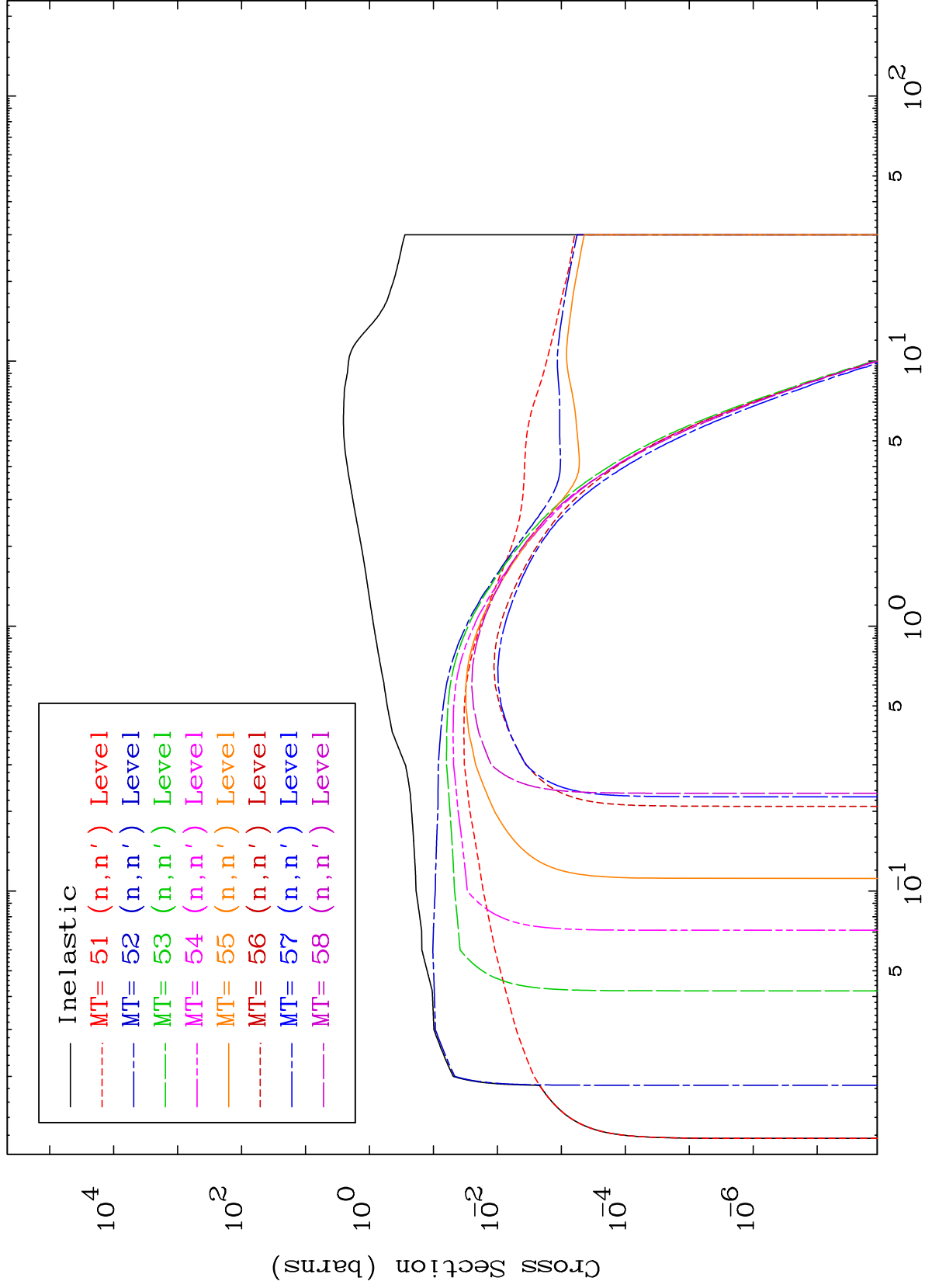


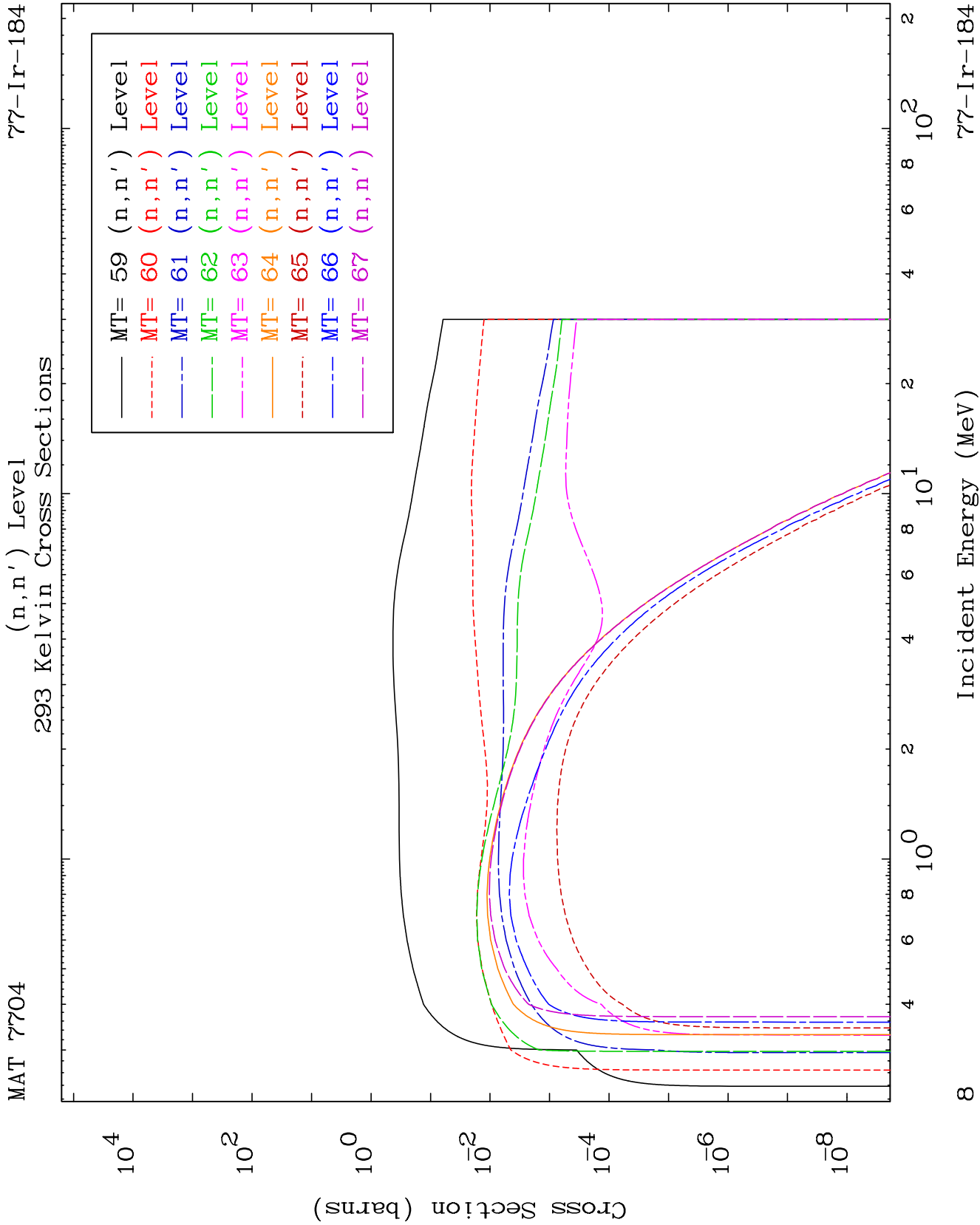


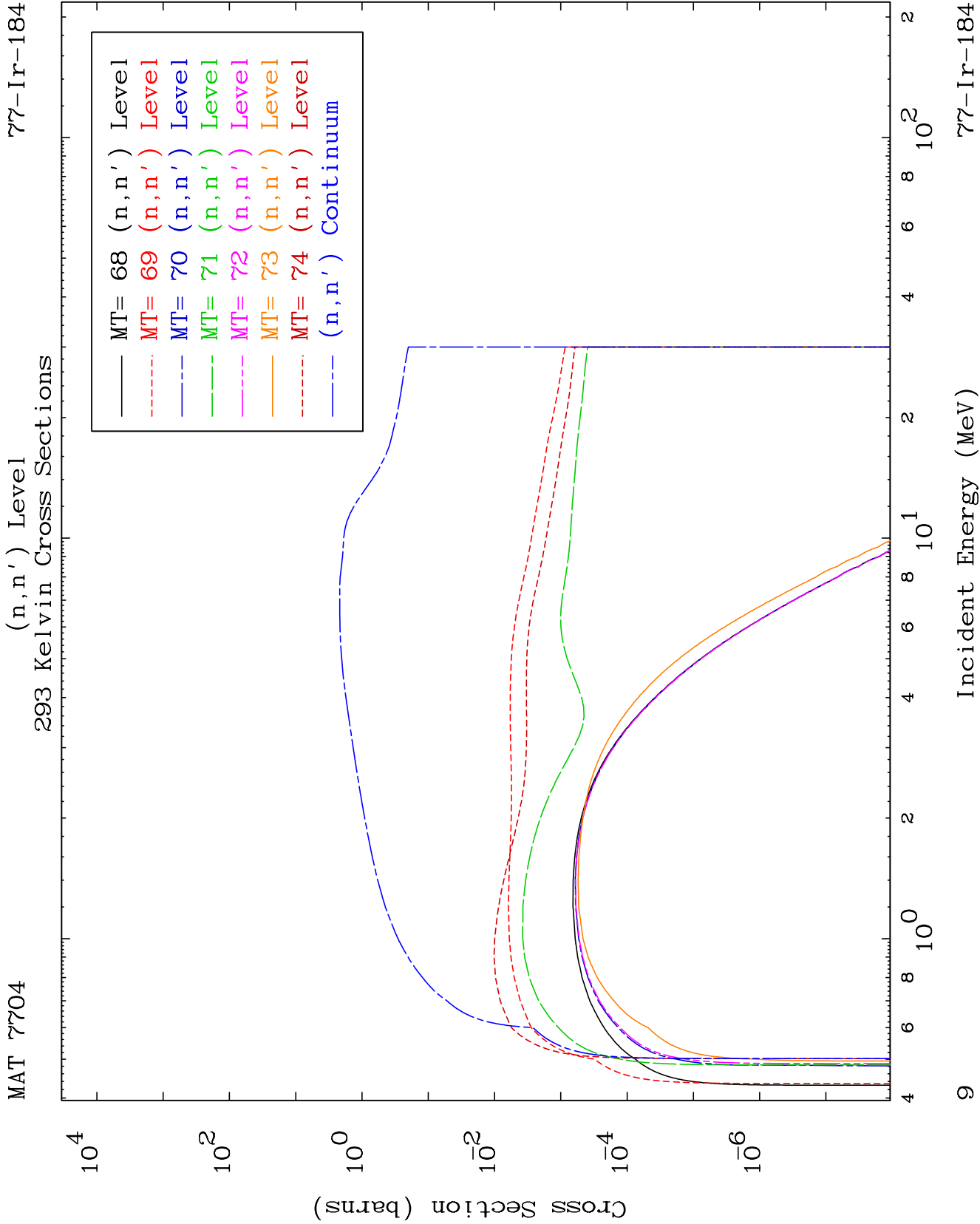


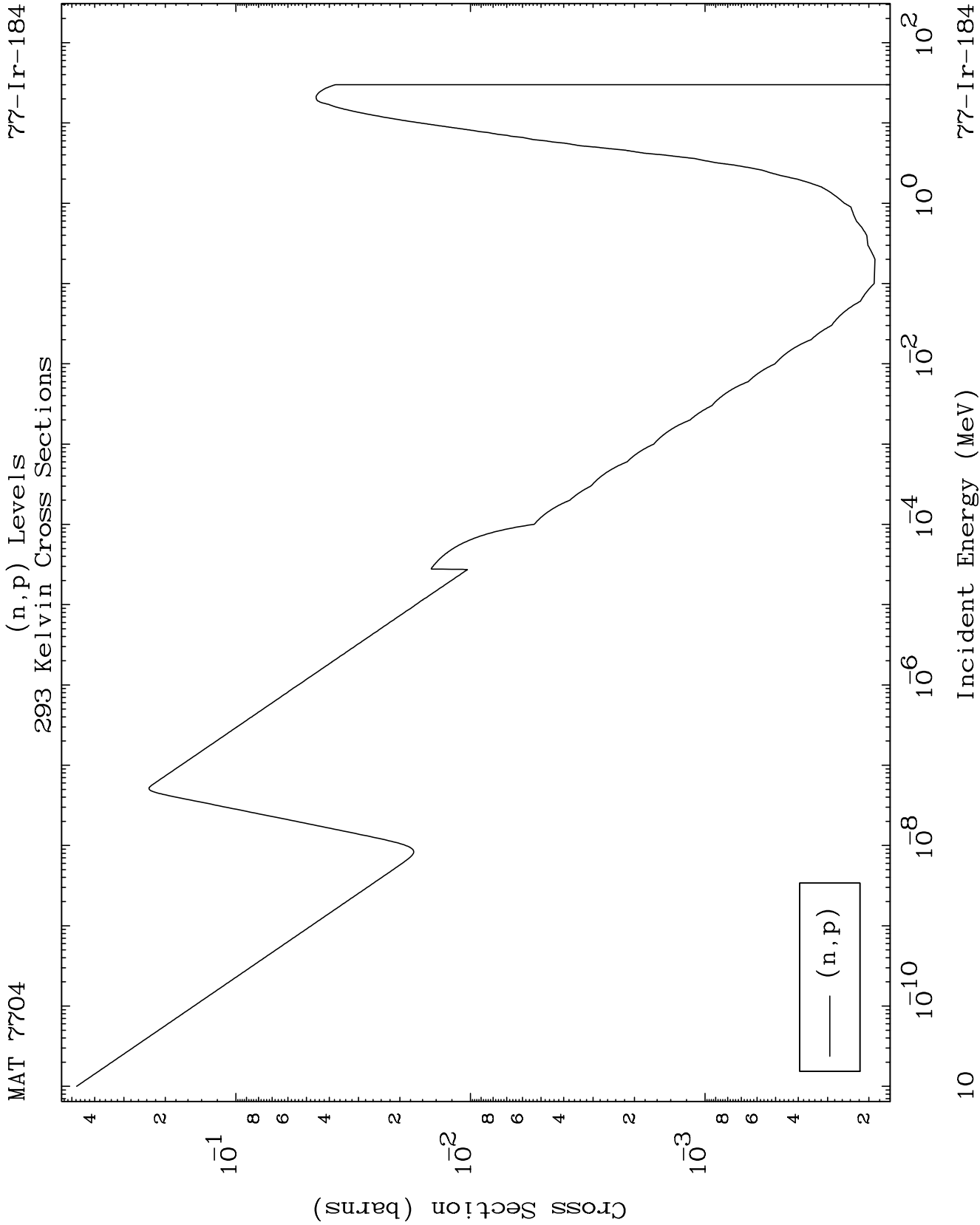


(n,n') Level
293 Kelvin Cross Sections





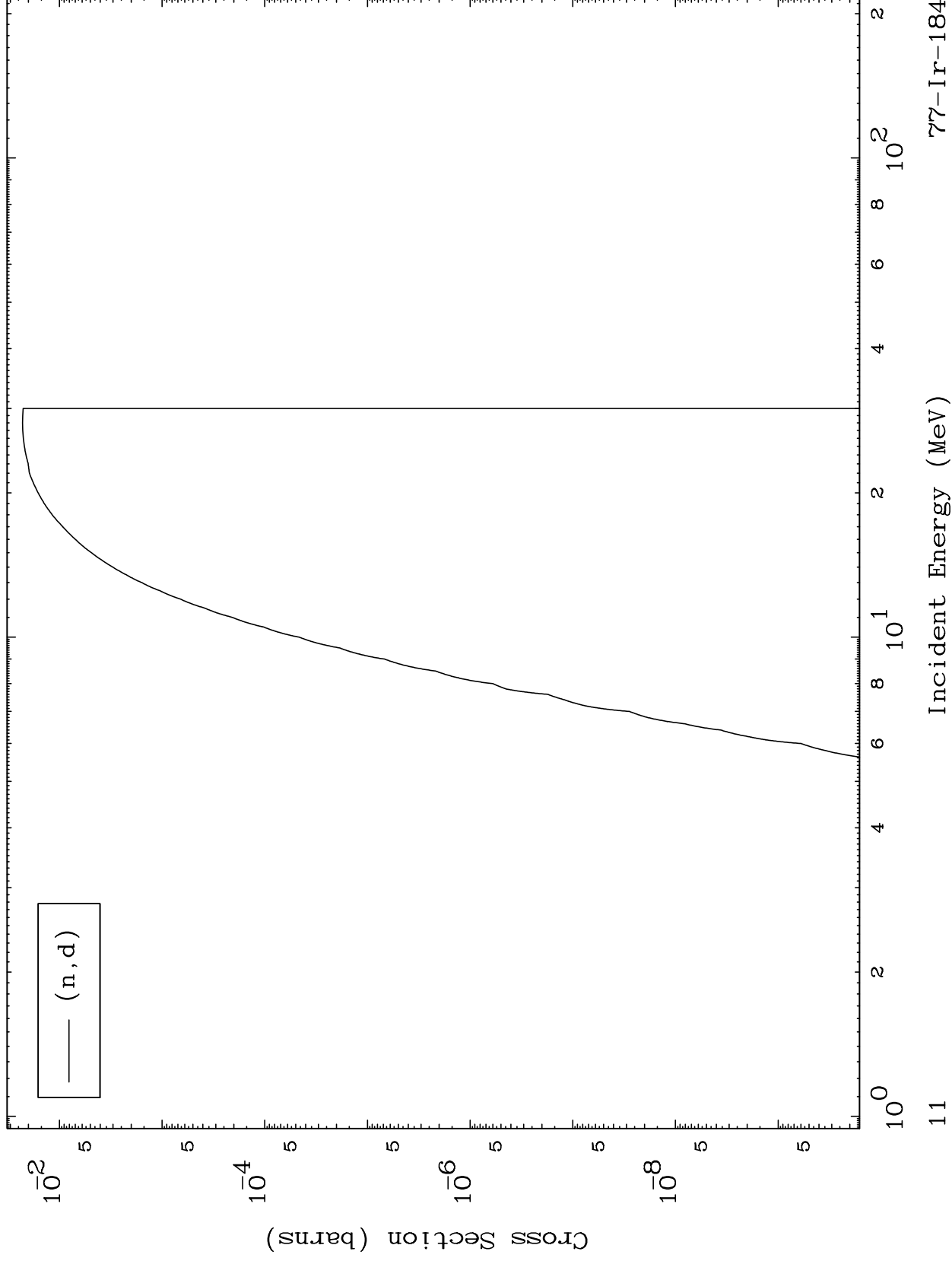


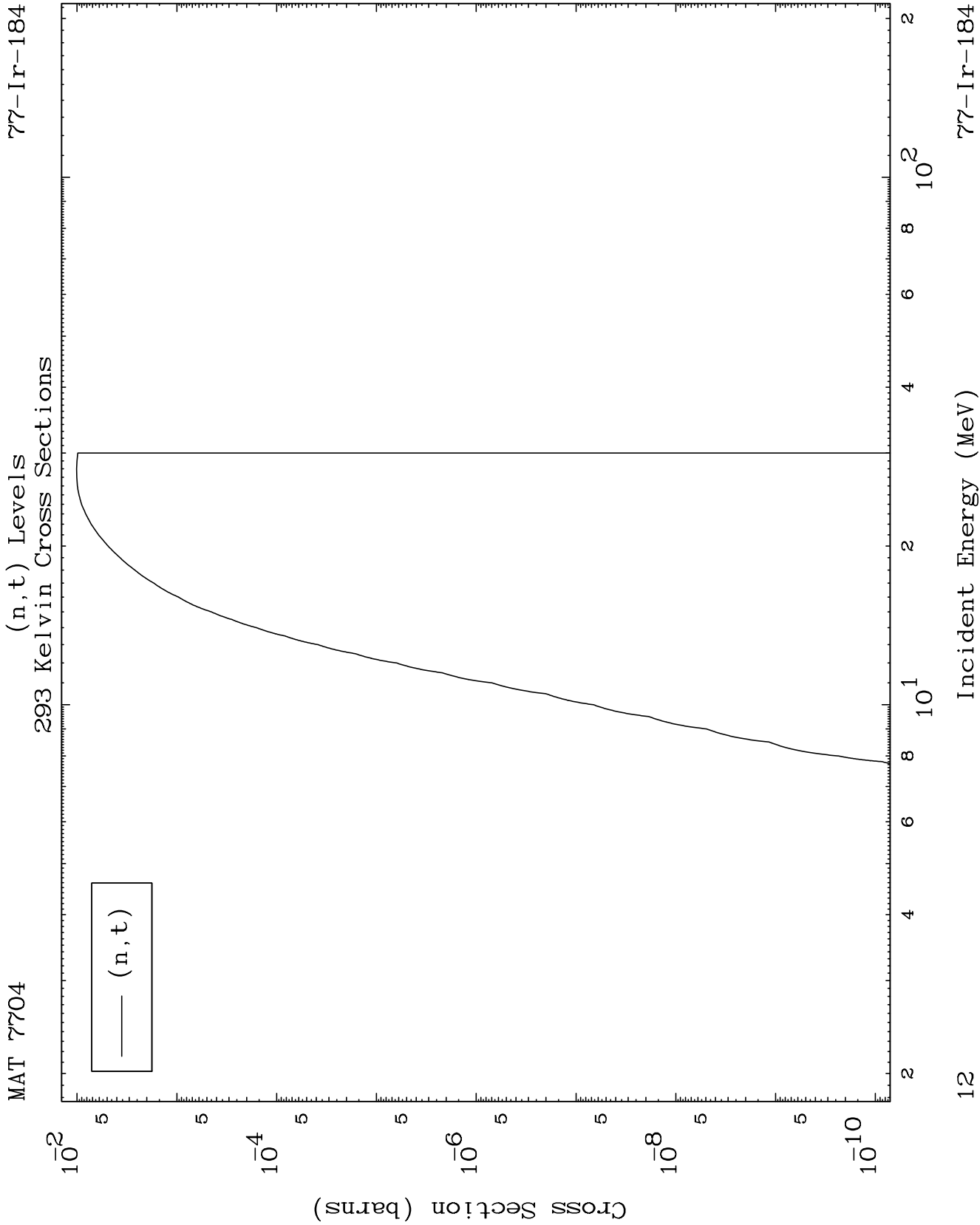


MAT 7704

77-Ir-184

(n,d) Levels
293 Kelvin Cross Sections

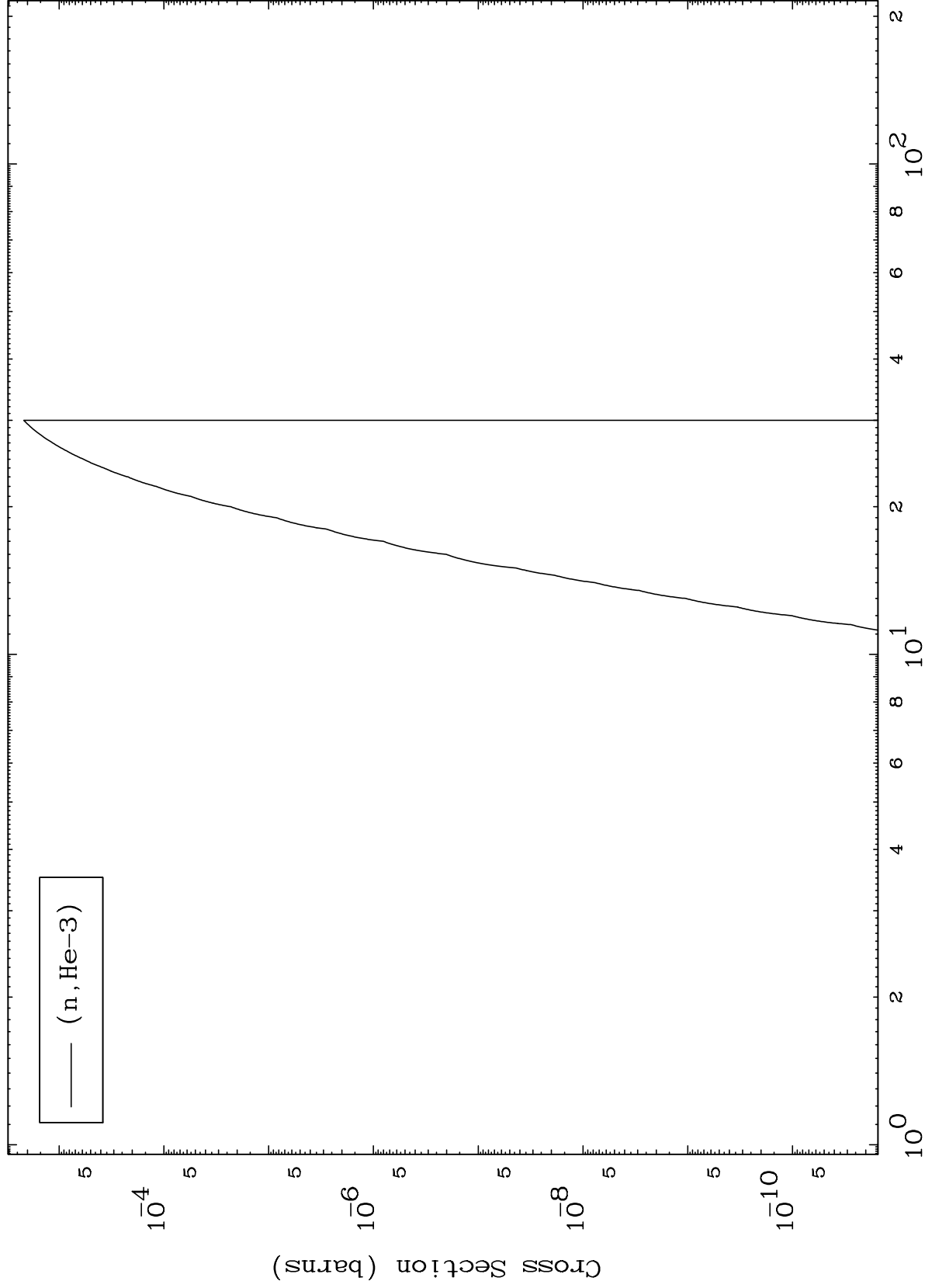




MAT 7704

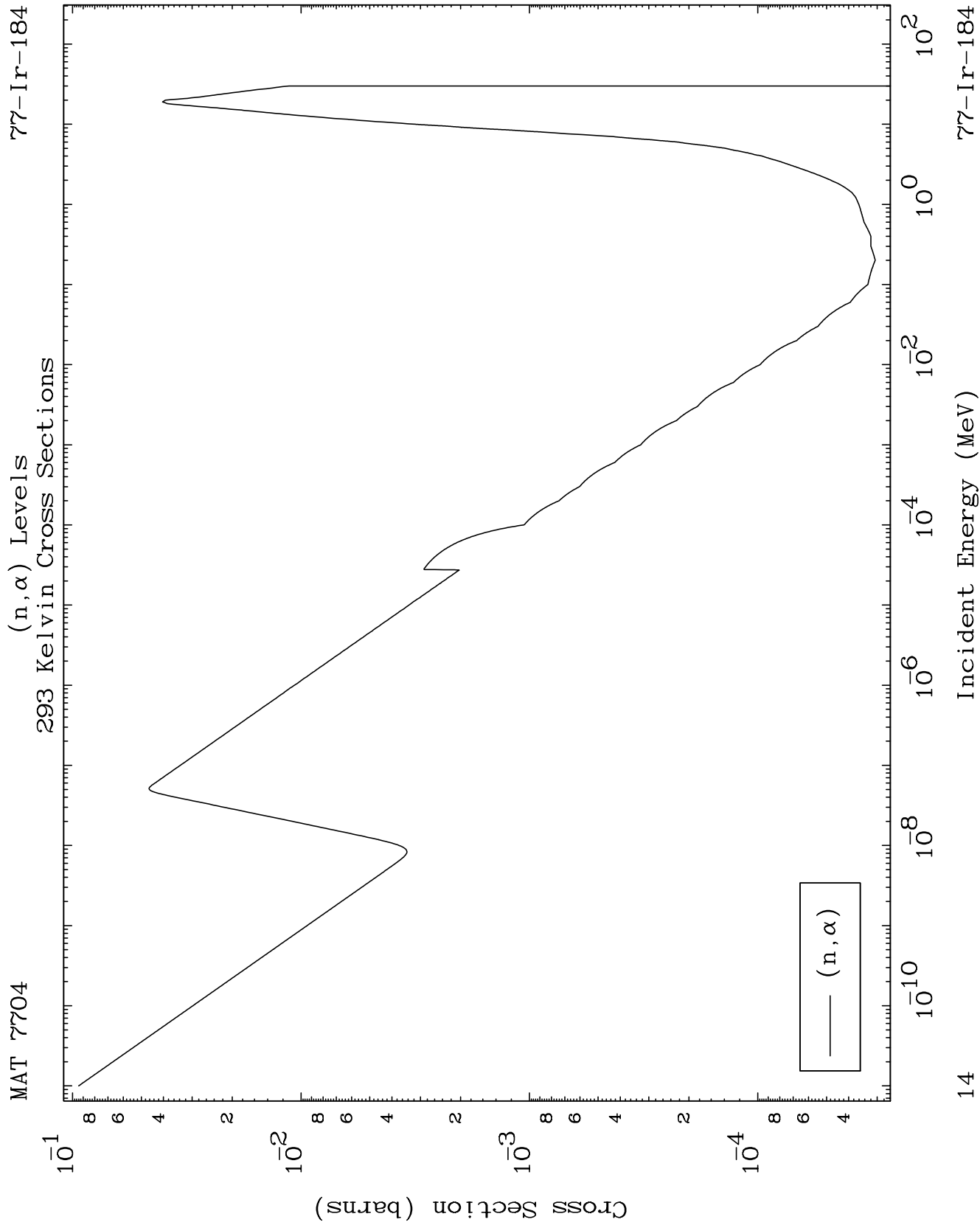
(n,He3) Levels
293 Kelvin Cross Sections

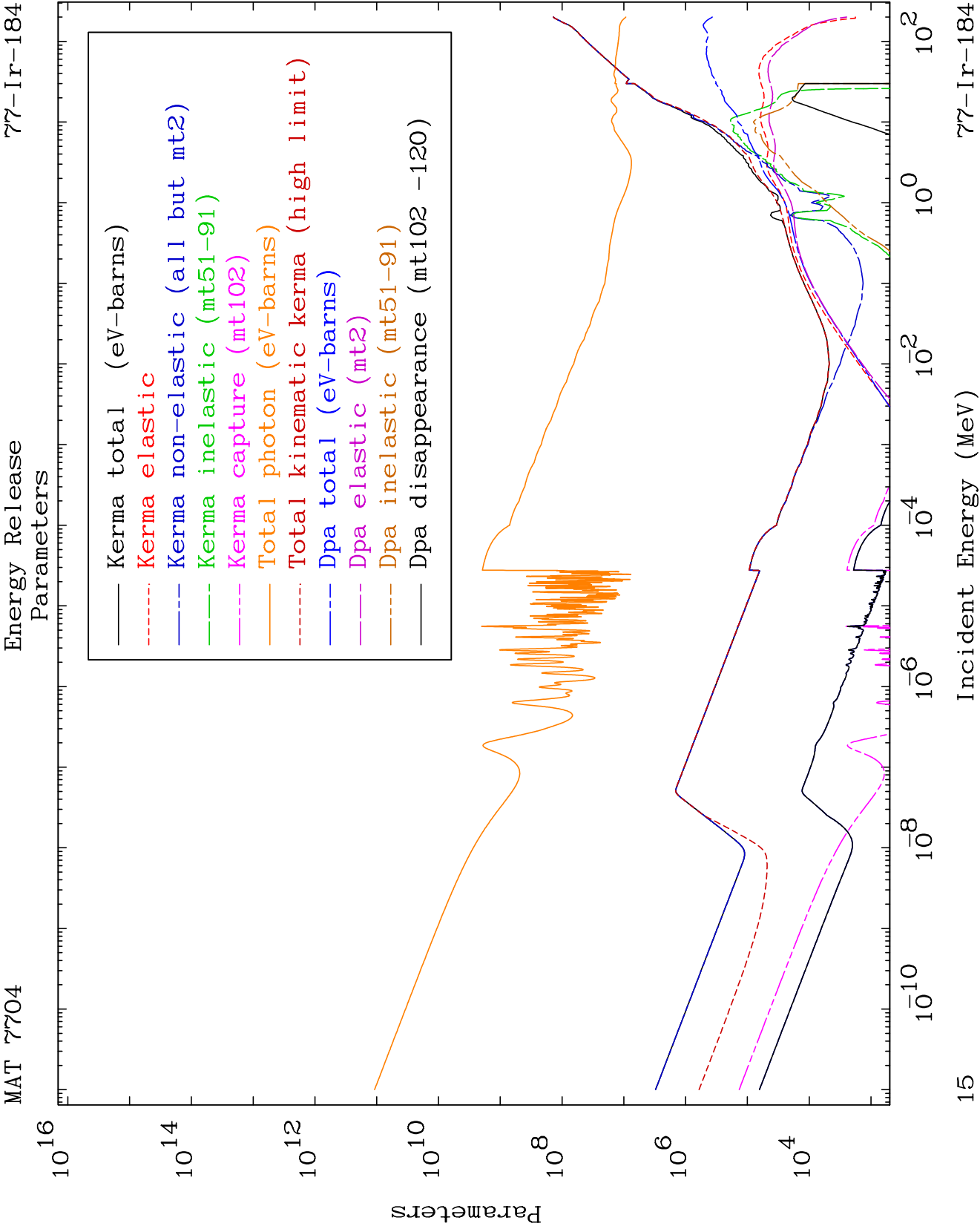
77-Ir-184

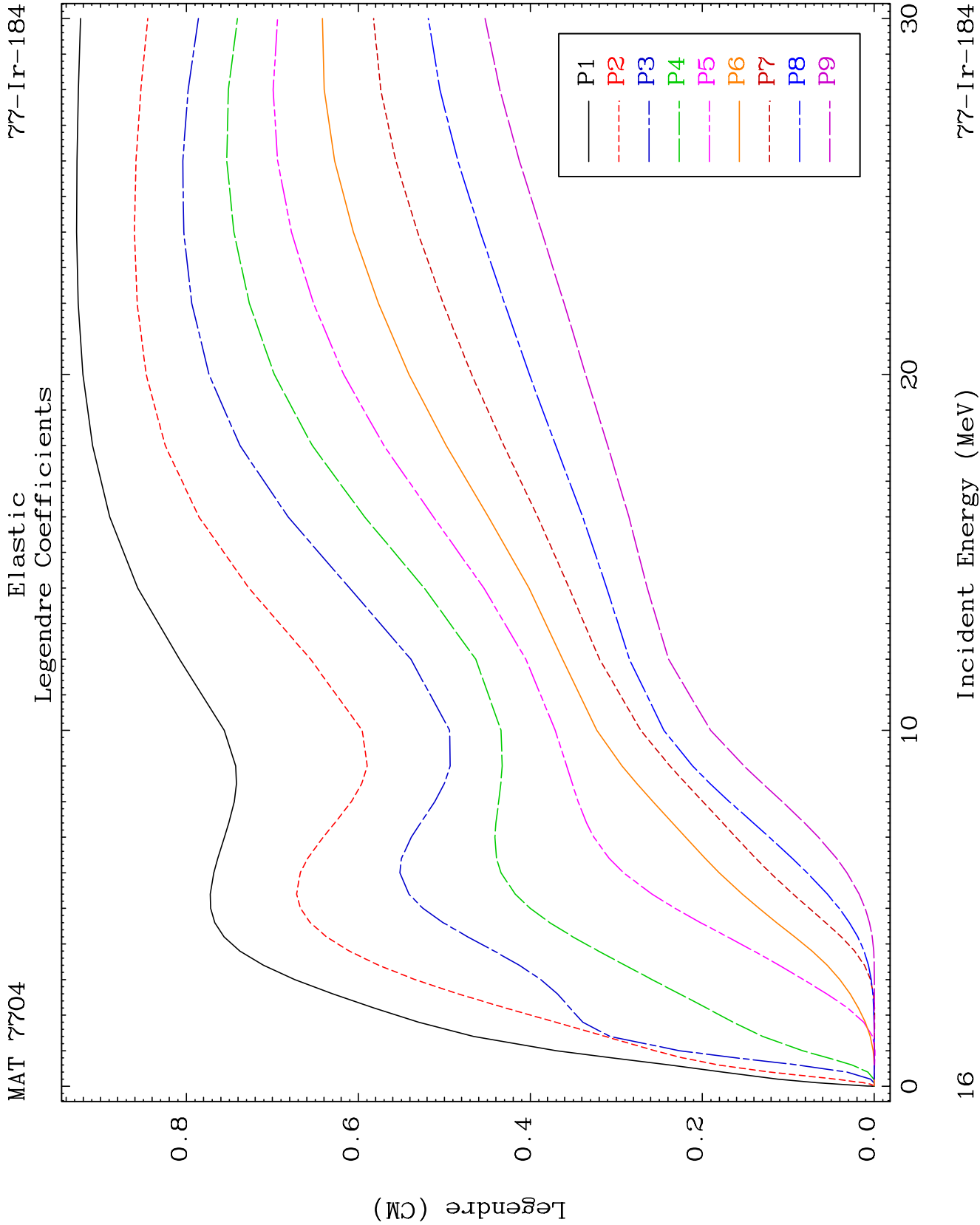


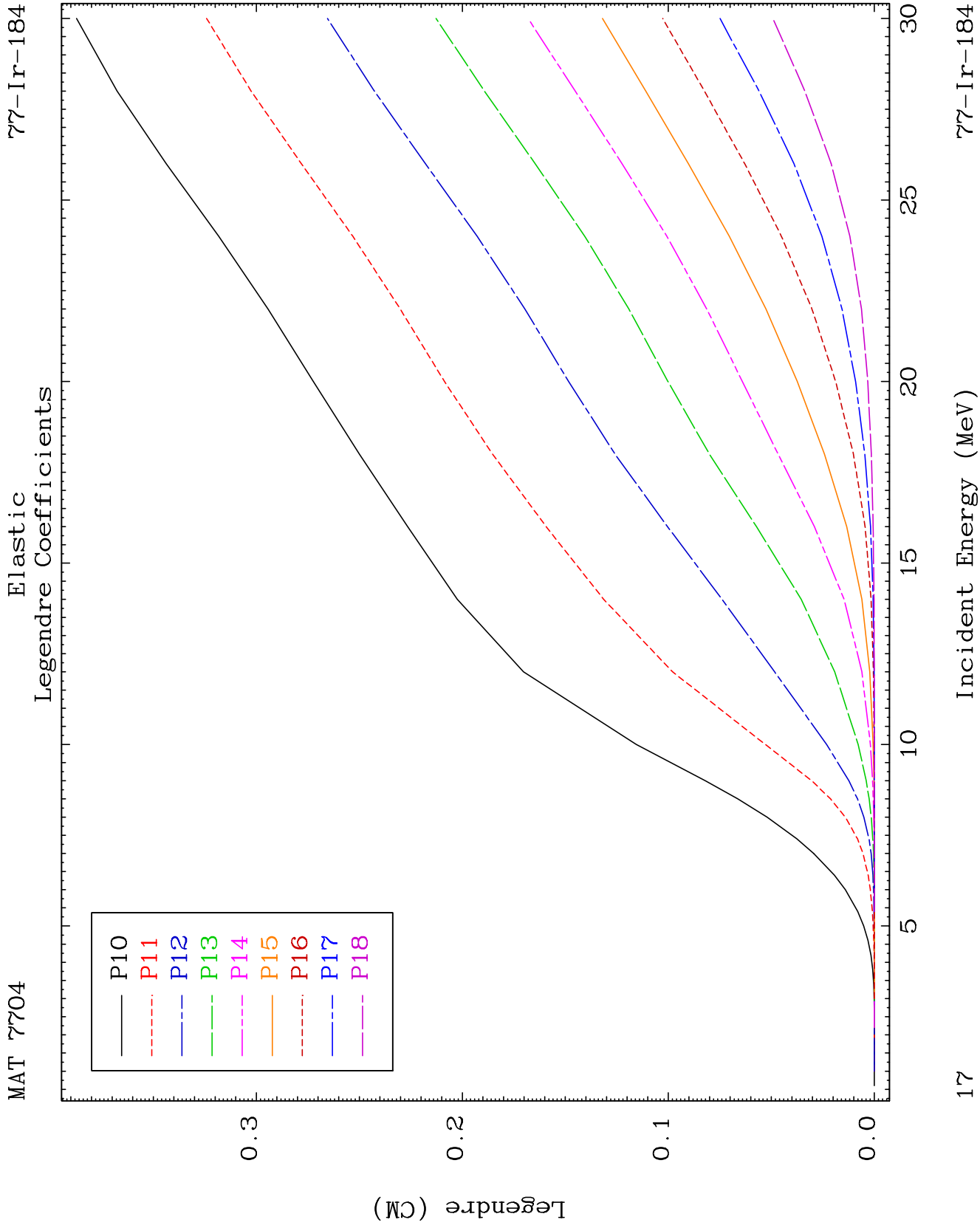
Incident Energy (MeV)

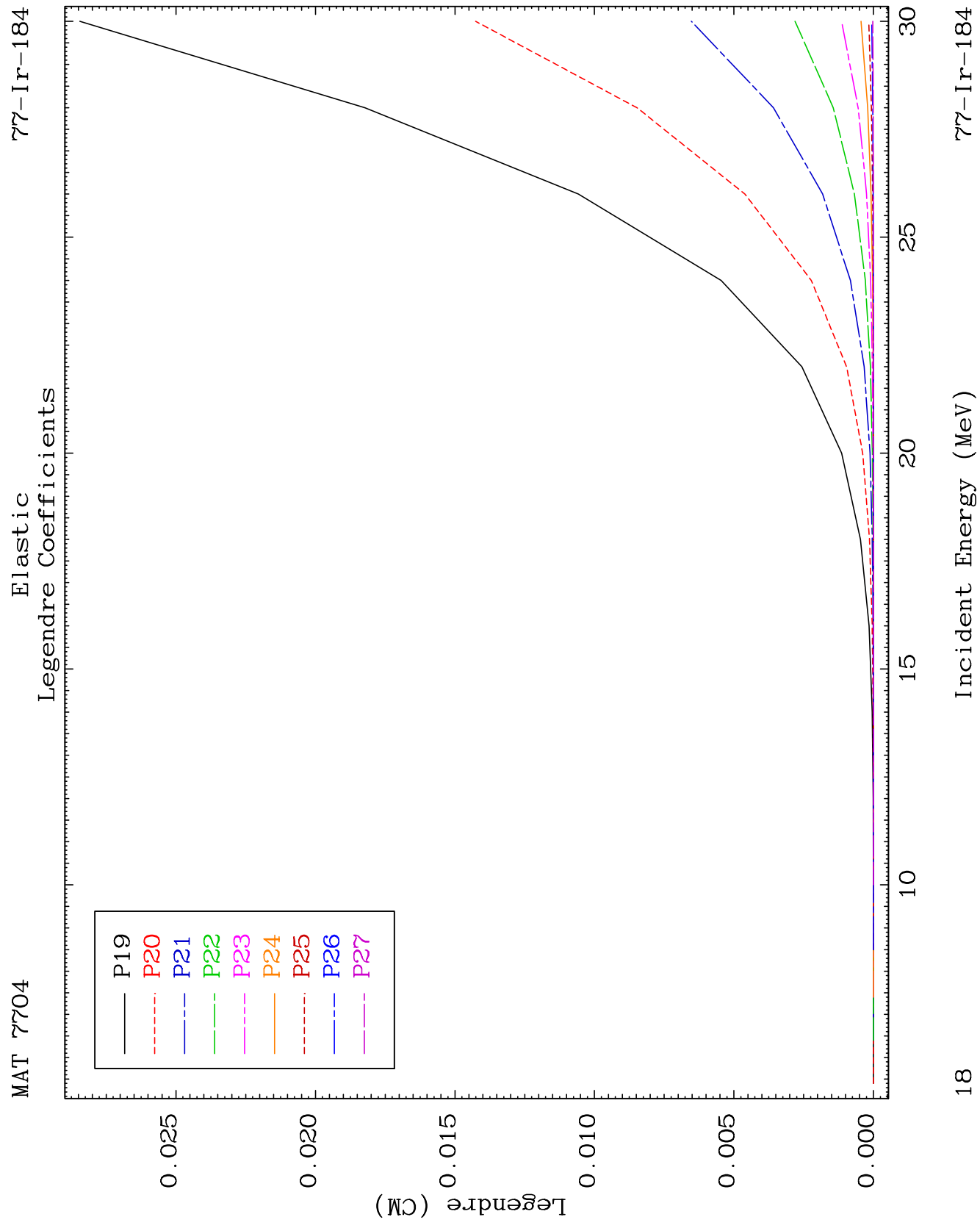
77-Ir-184

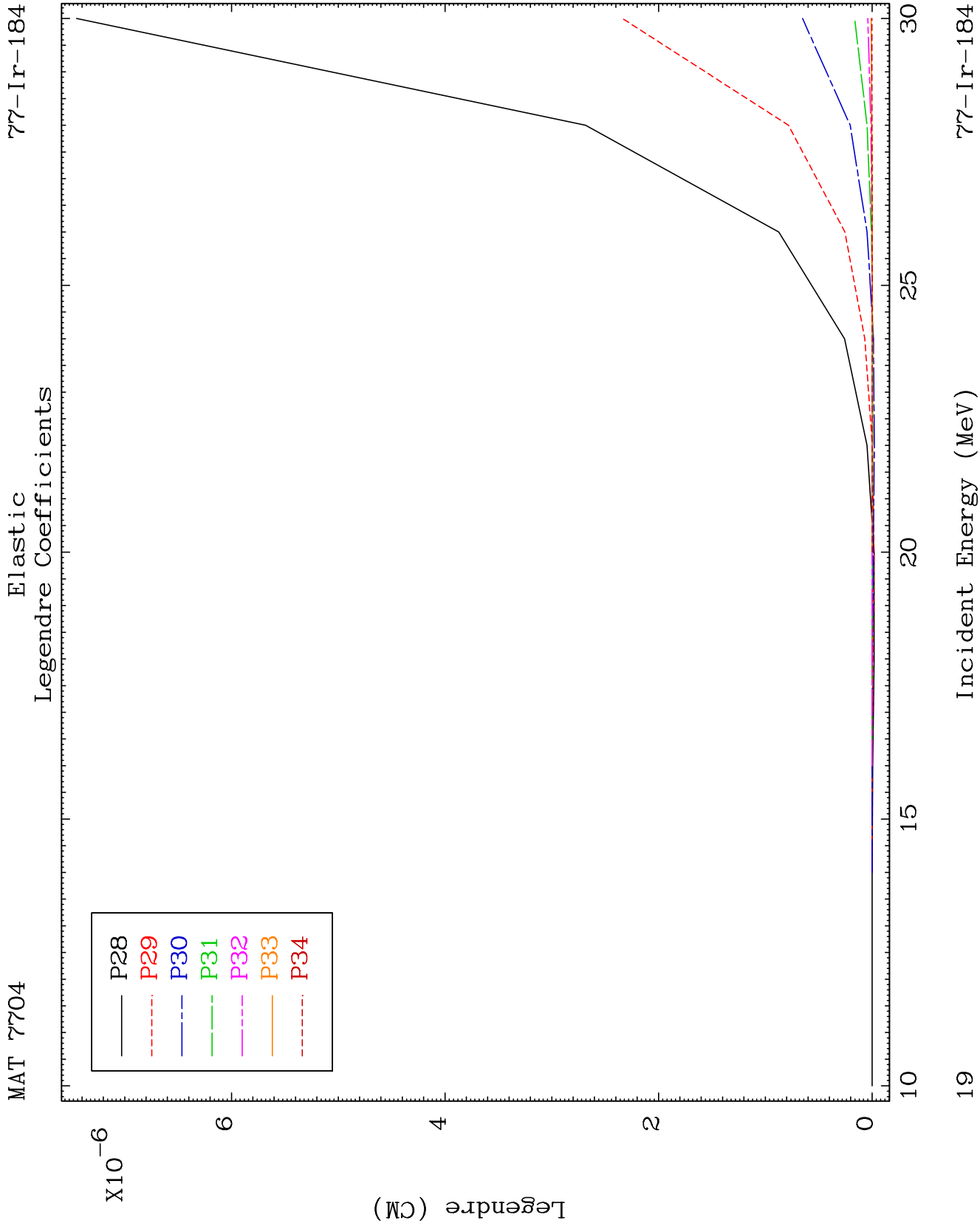








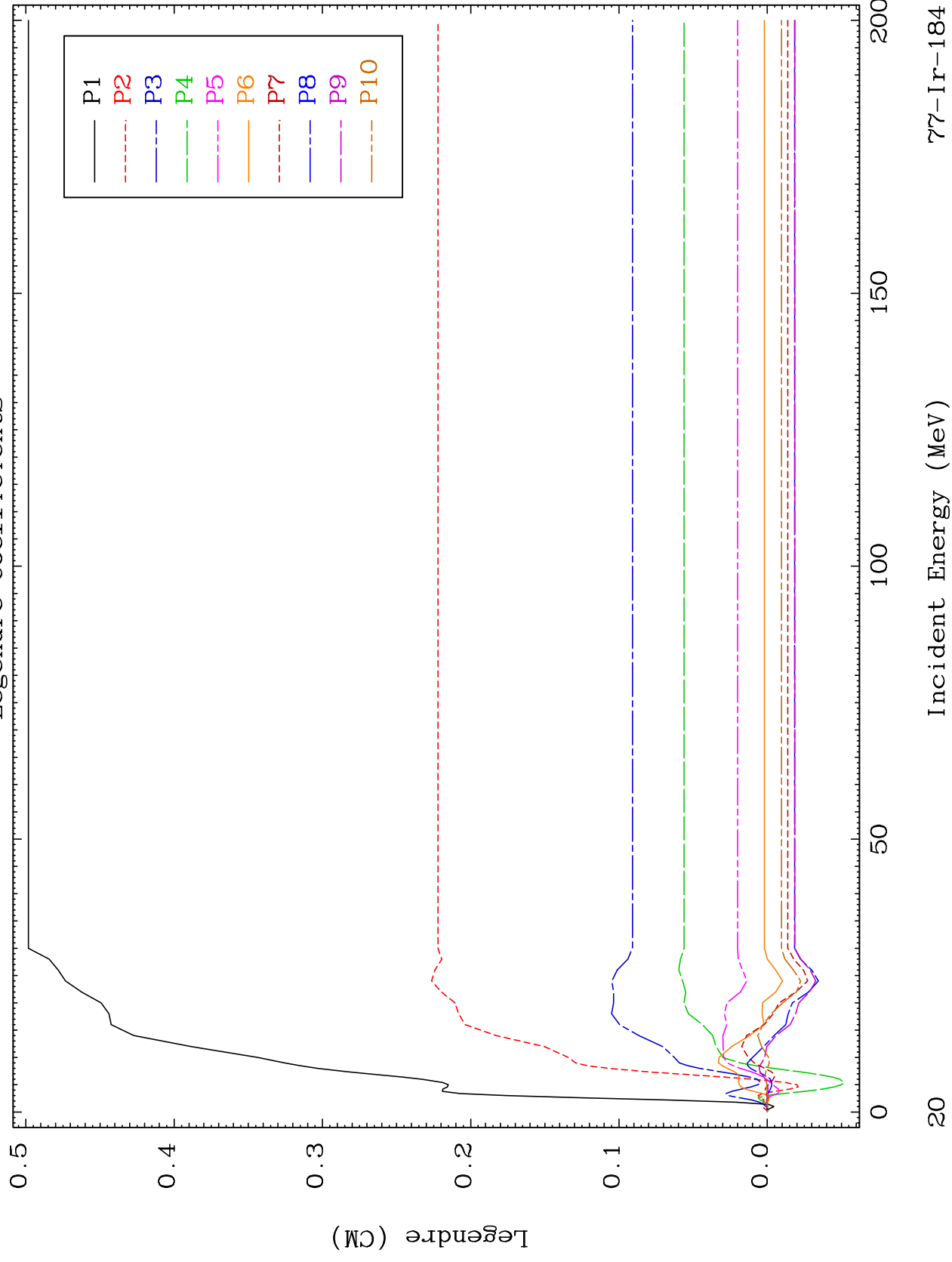




MAT 2704

MT= 51 (n, n') Level
Legendre Coefficients

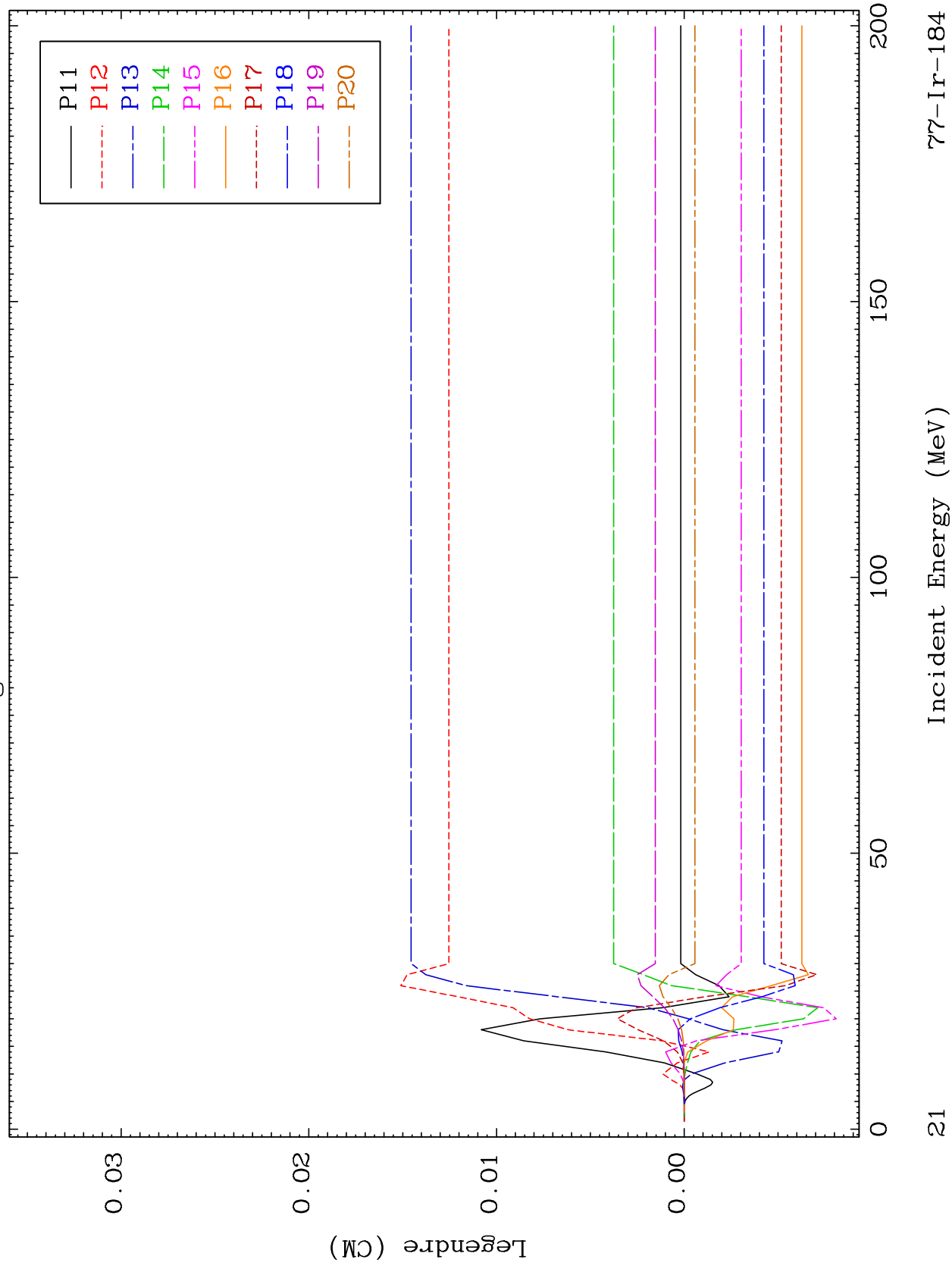
77-Ir-184

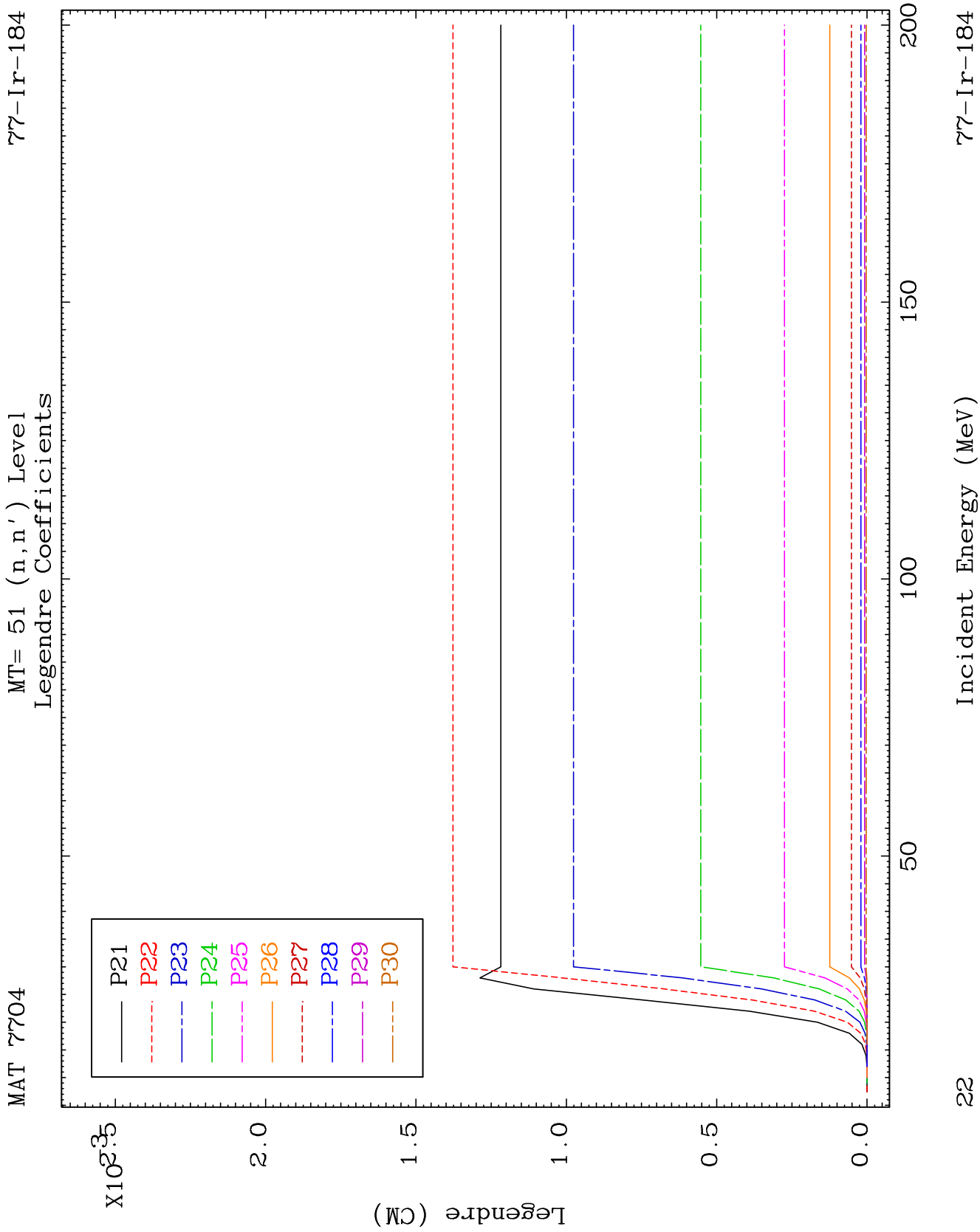


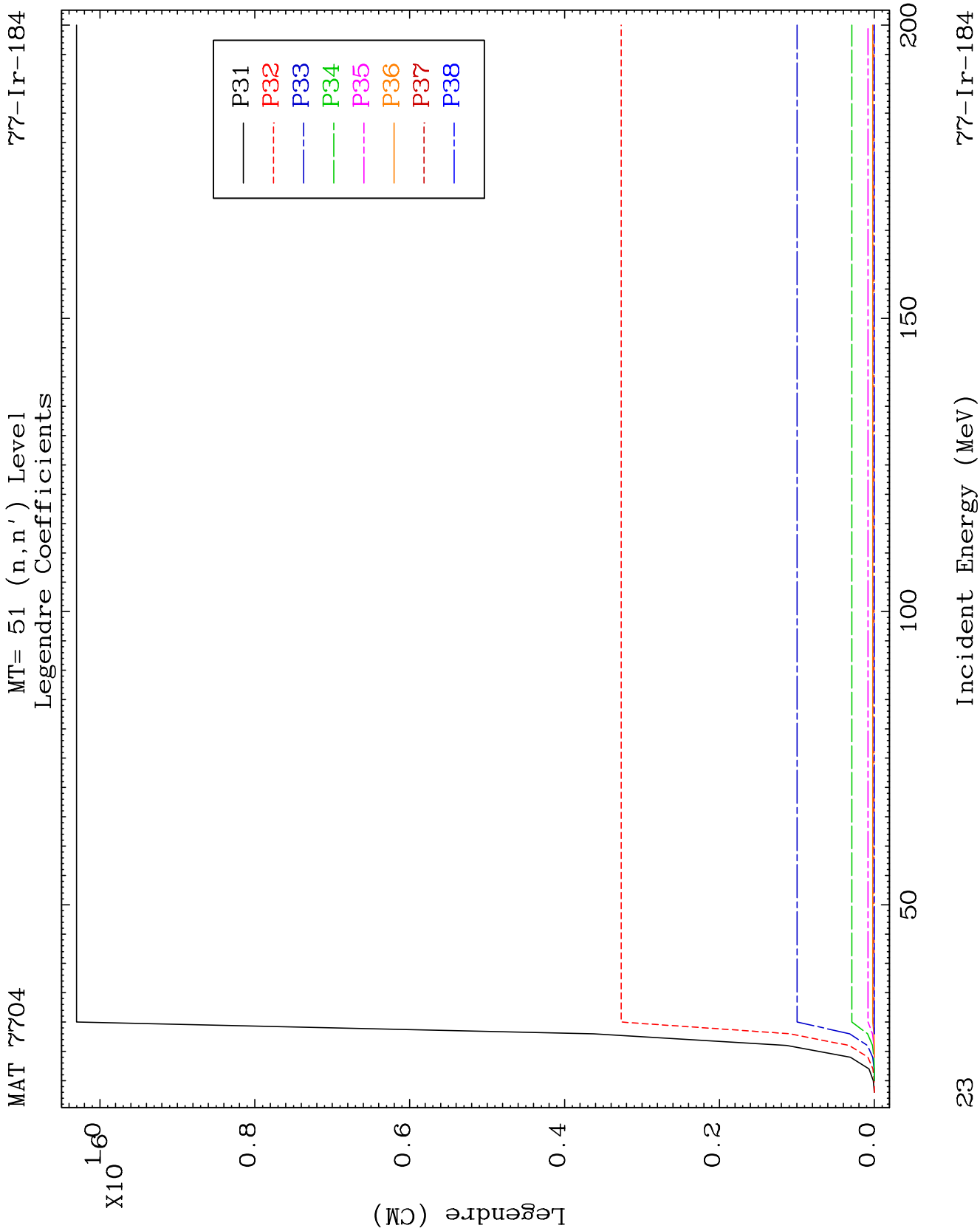
MAT 2704

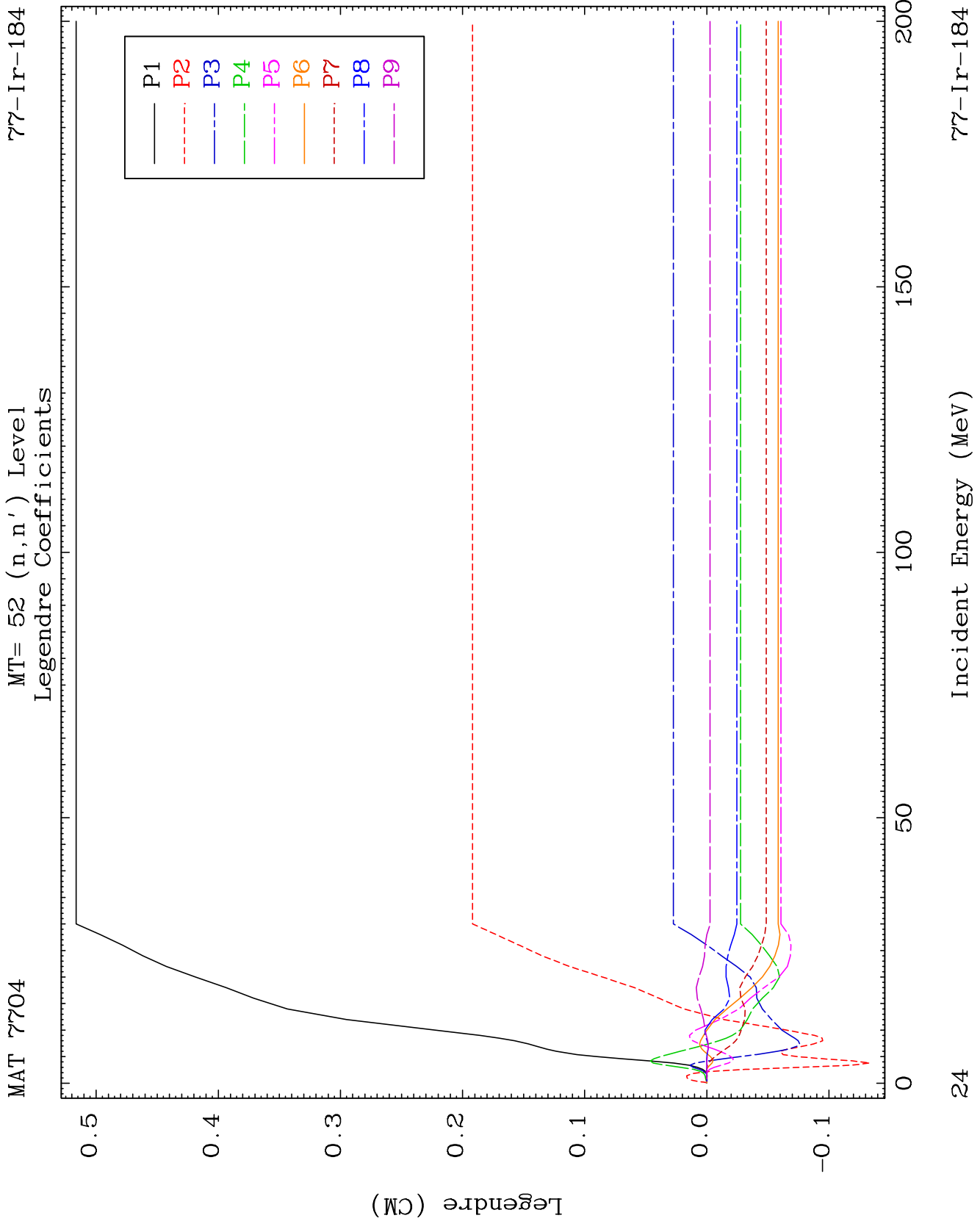
MT= 51 (n, n') Level
Legendre Coefficients

77-Ir-184





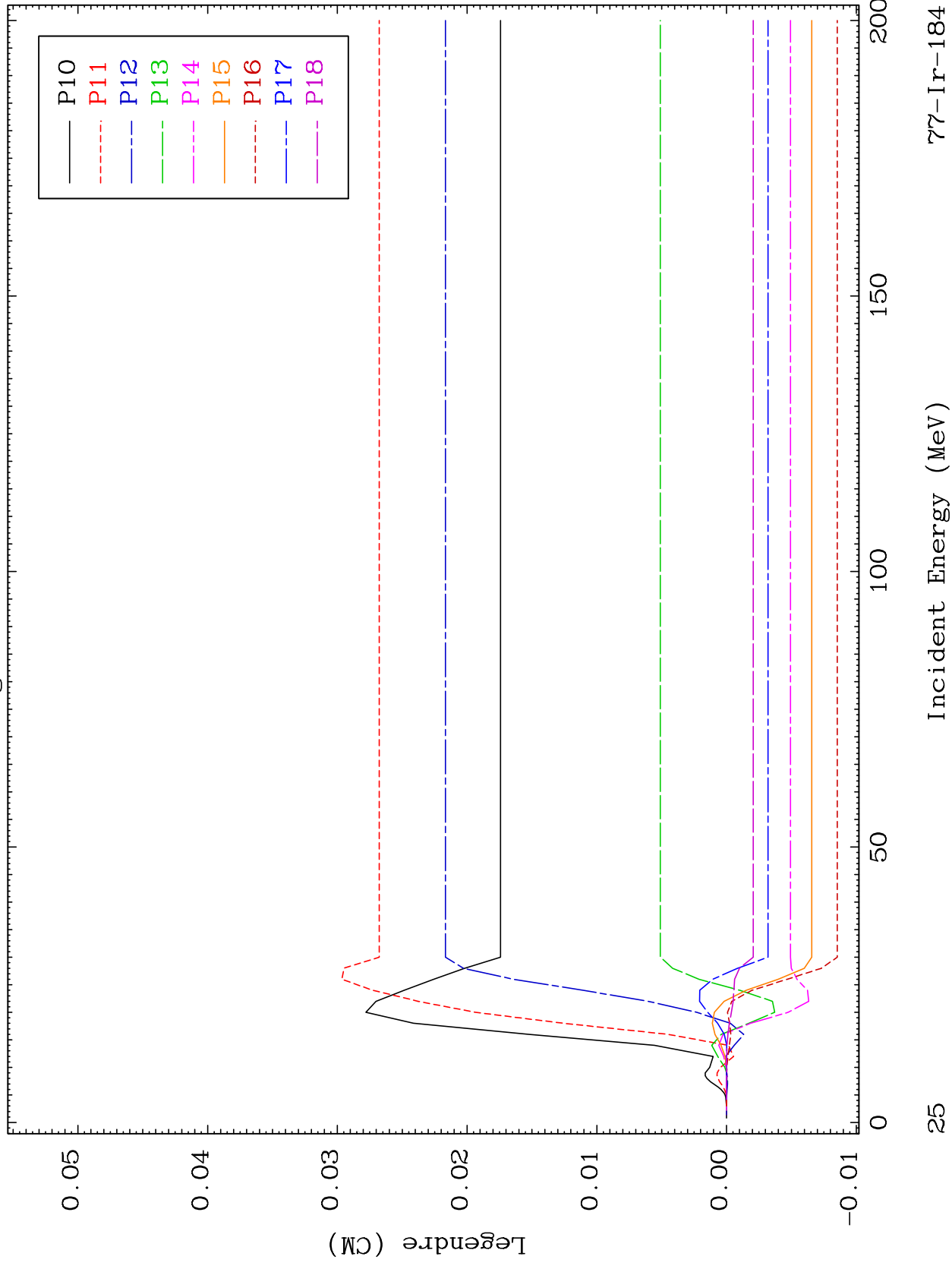


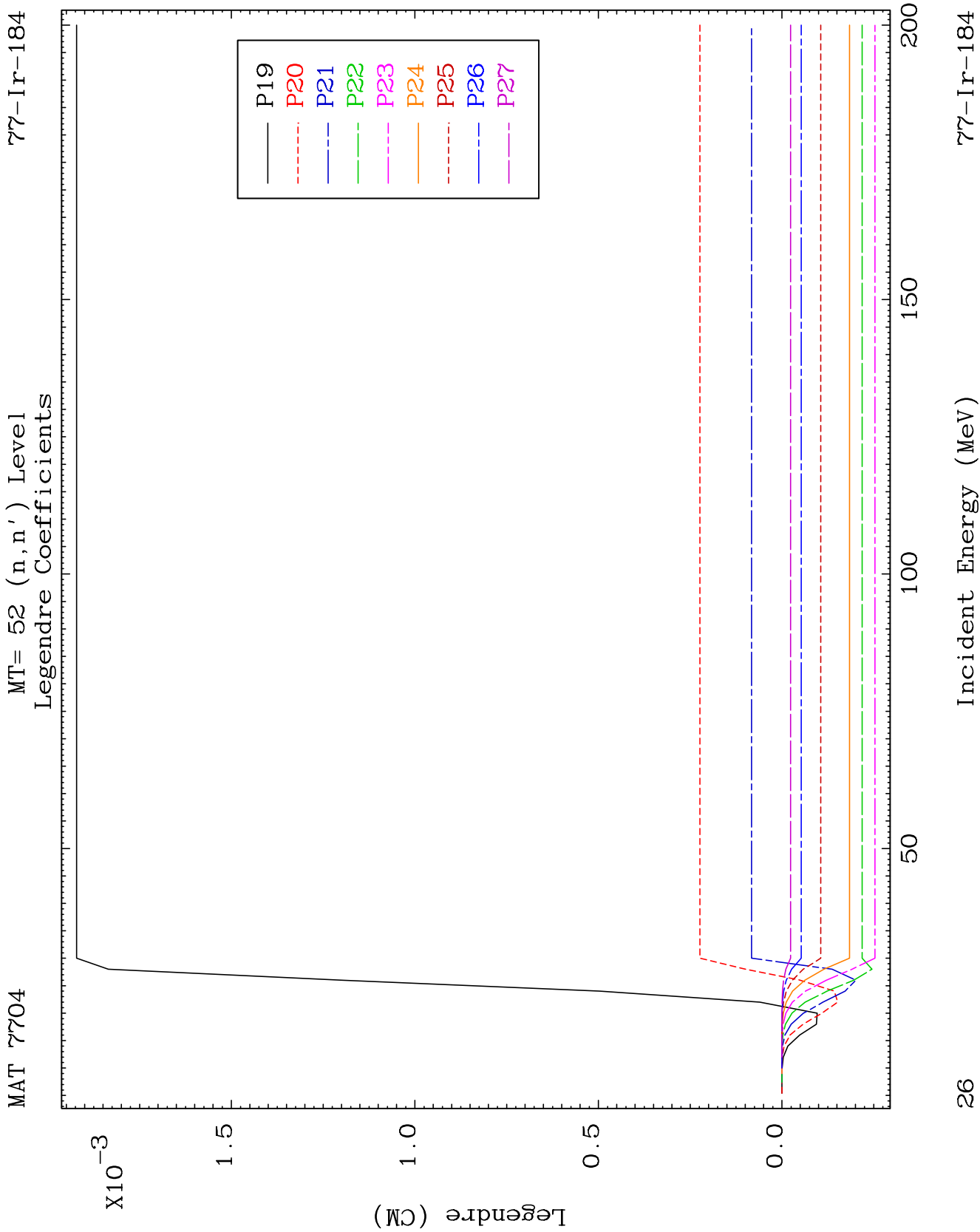


MAT 2704

MT= 52 (n, n') Level
Legendre Coefficients

77-Ir-184



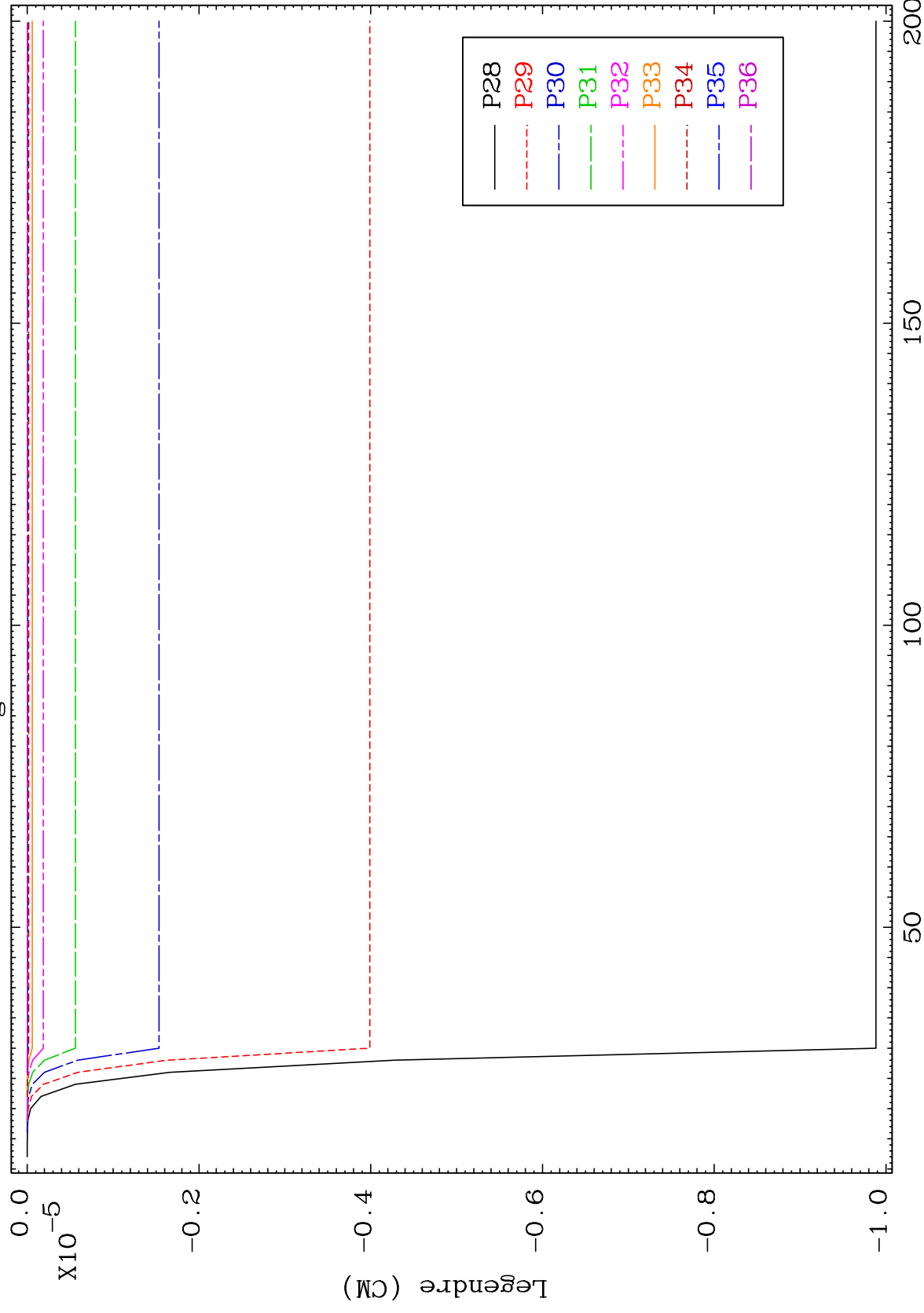


MAT 7704

MT= 52 (n, n') Level

77-Ir-184

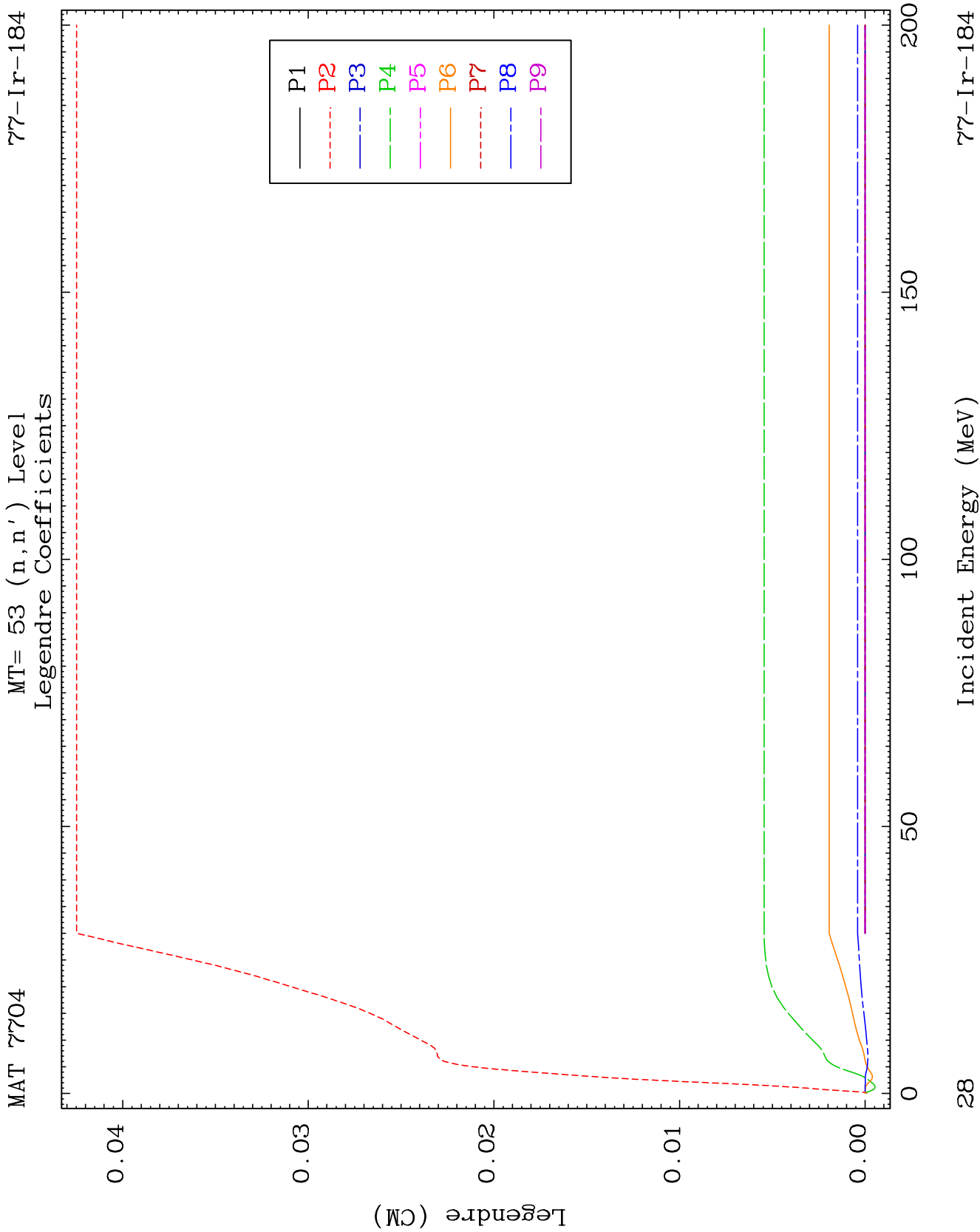
Legendre Coefficients

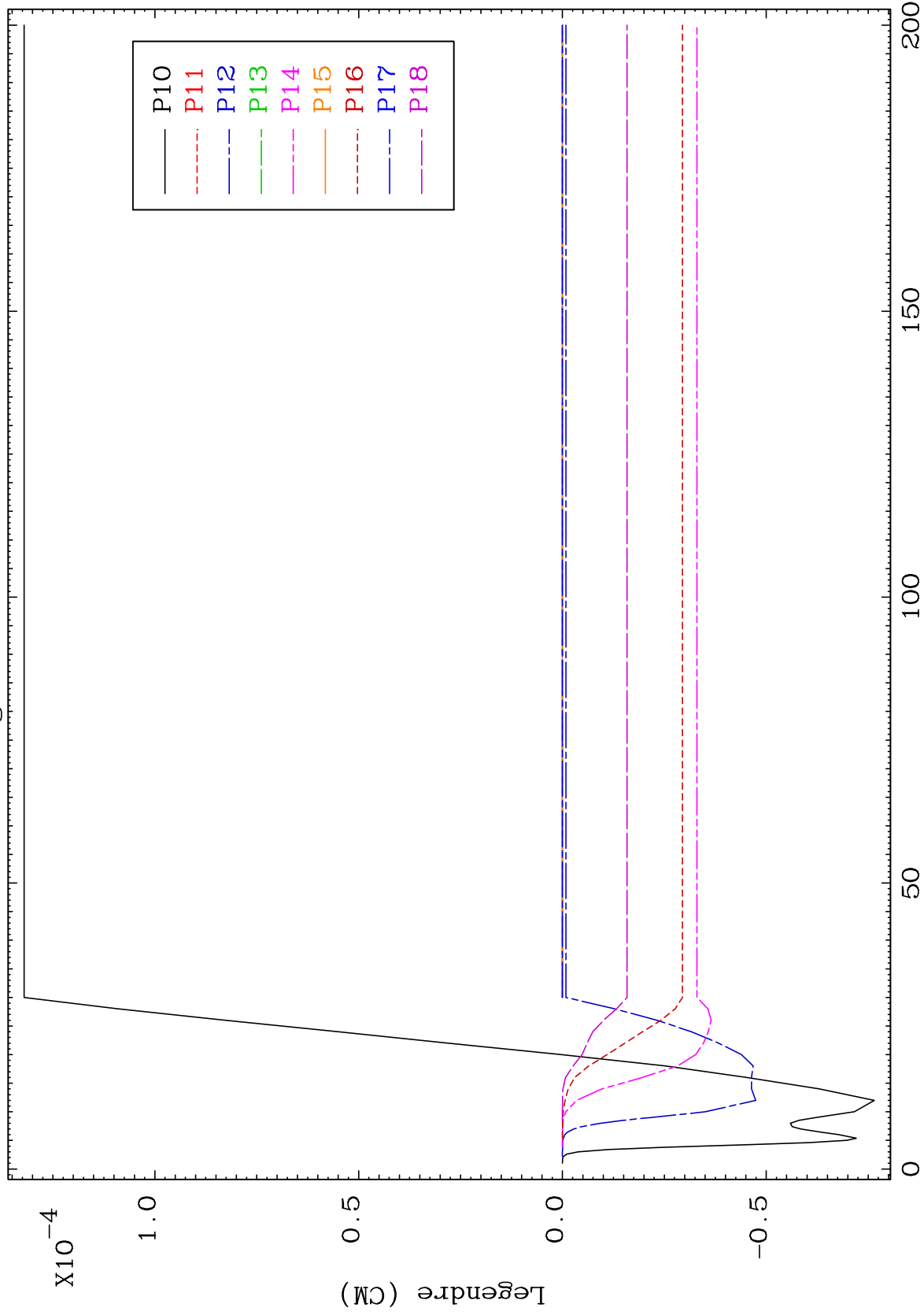


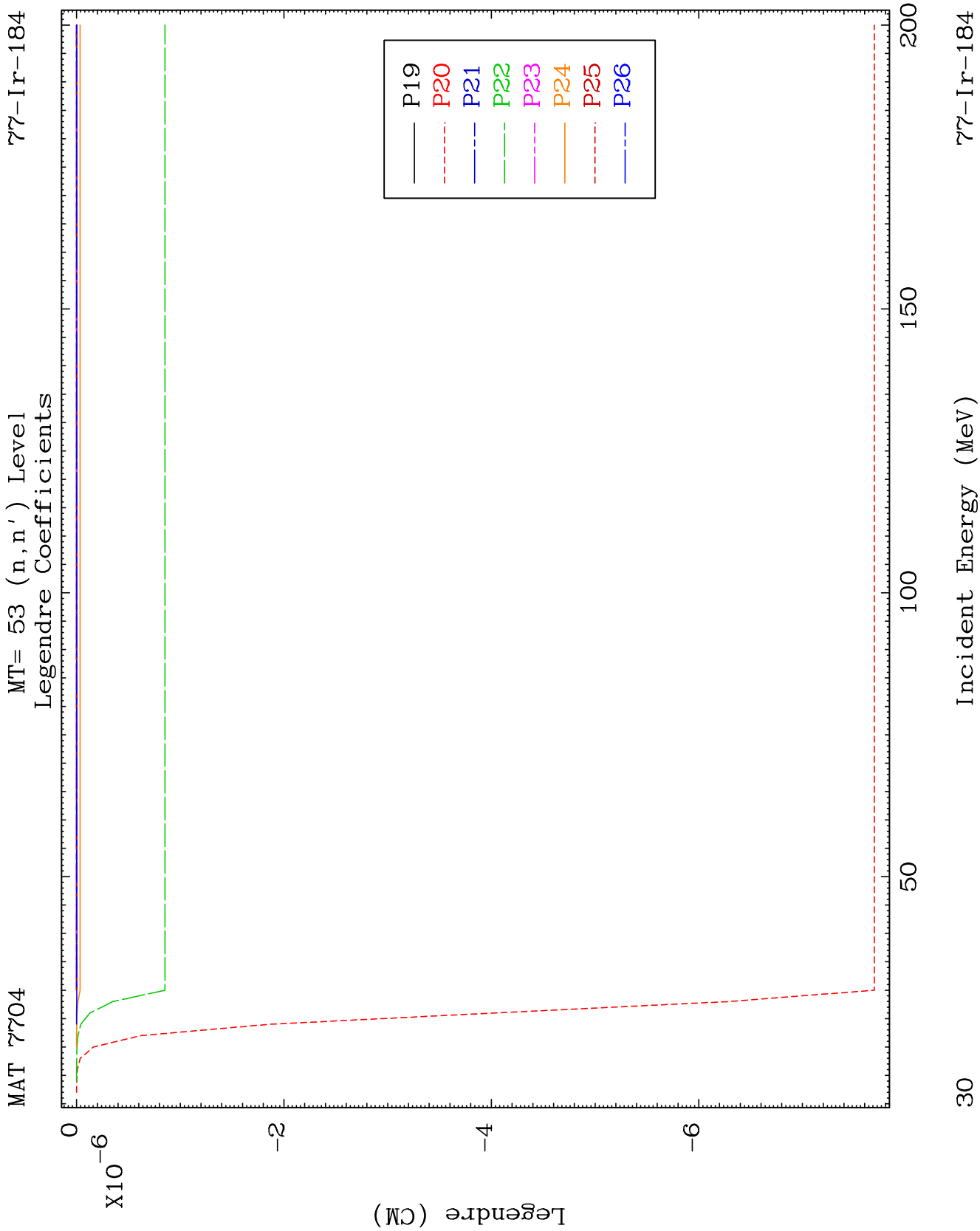
22

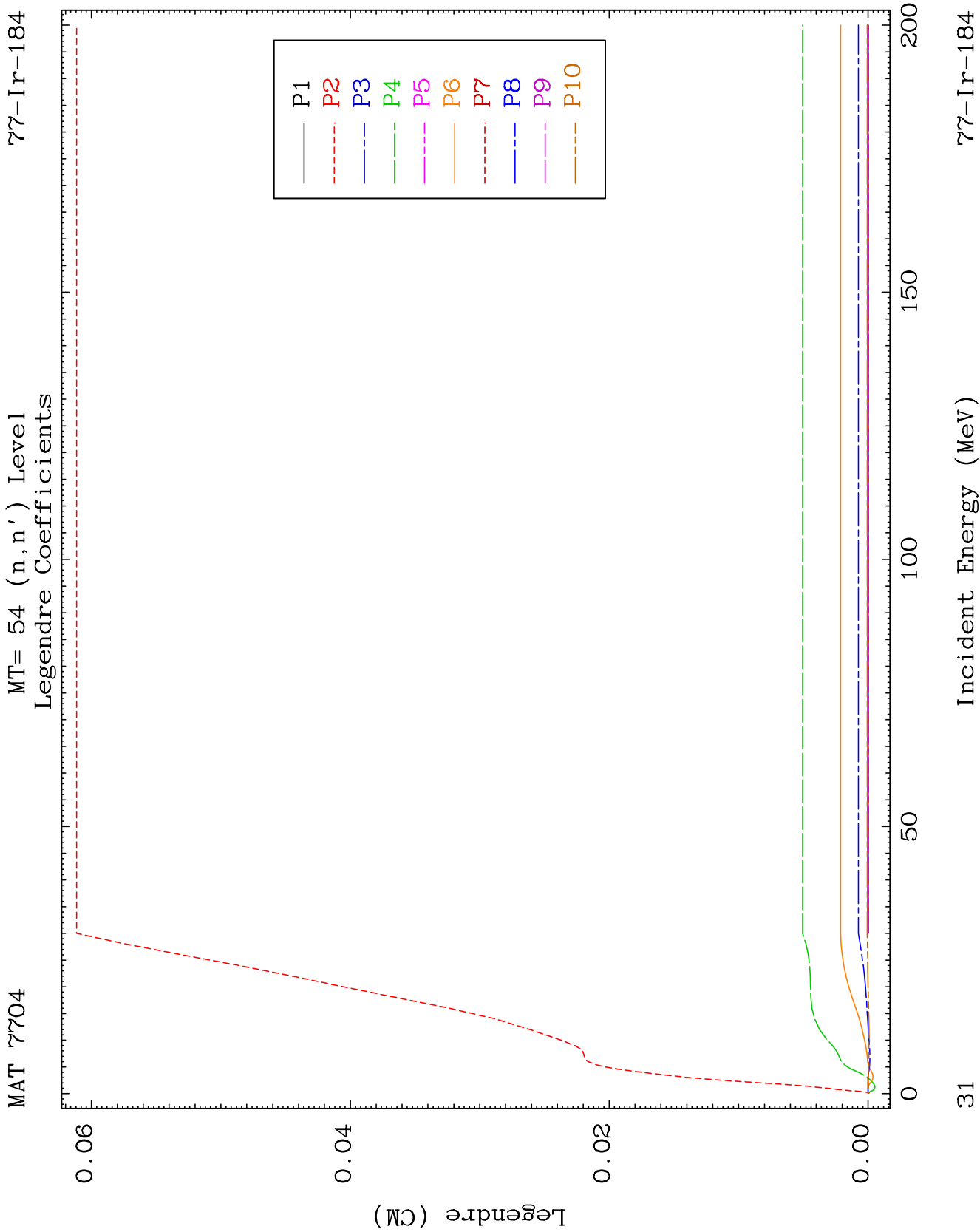
Incident Energy (MeV)

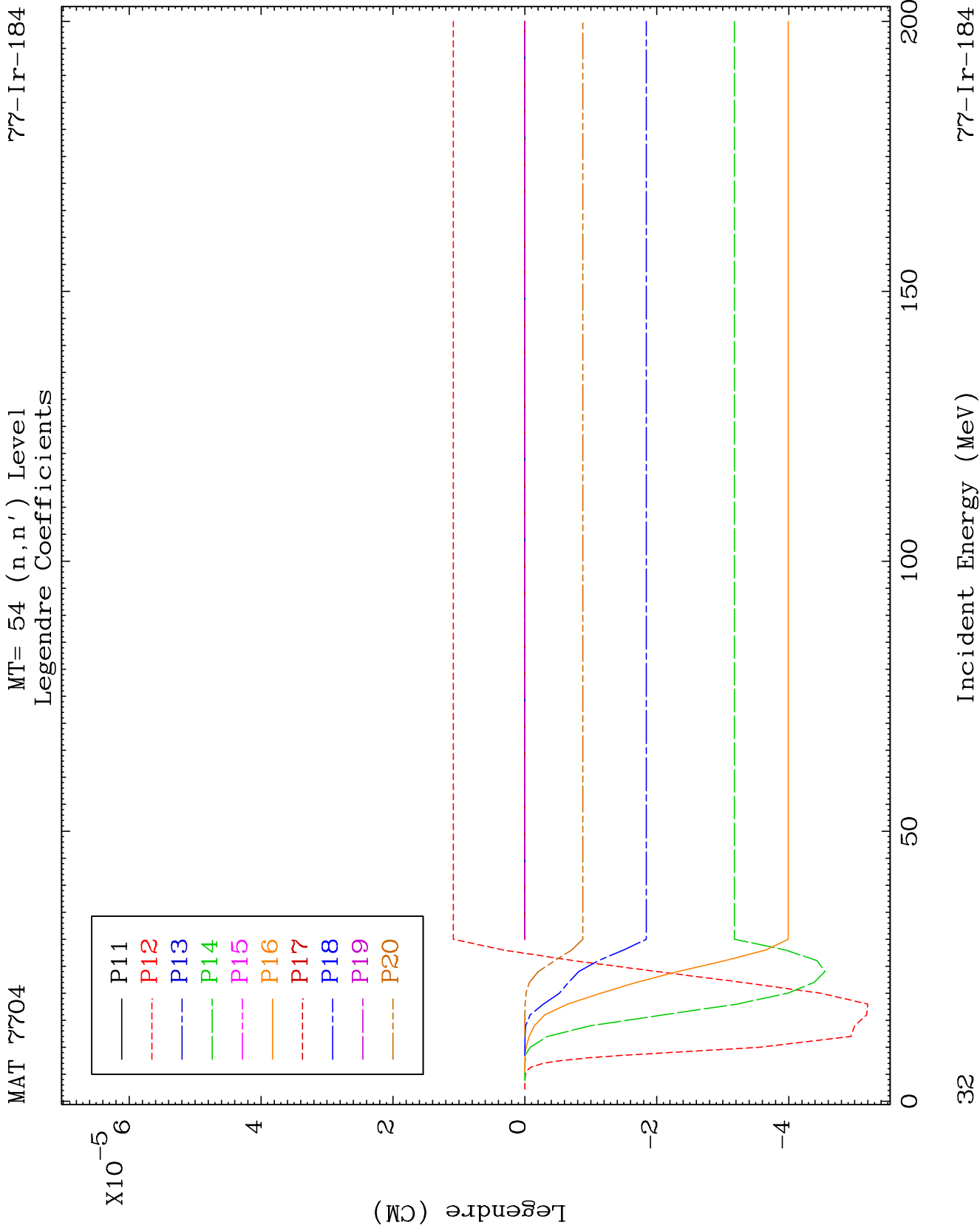
77-1r-184

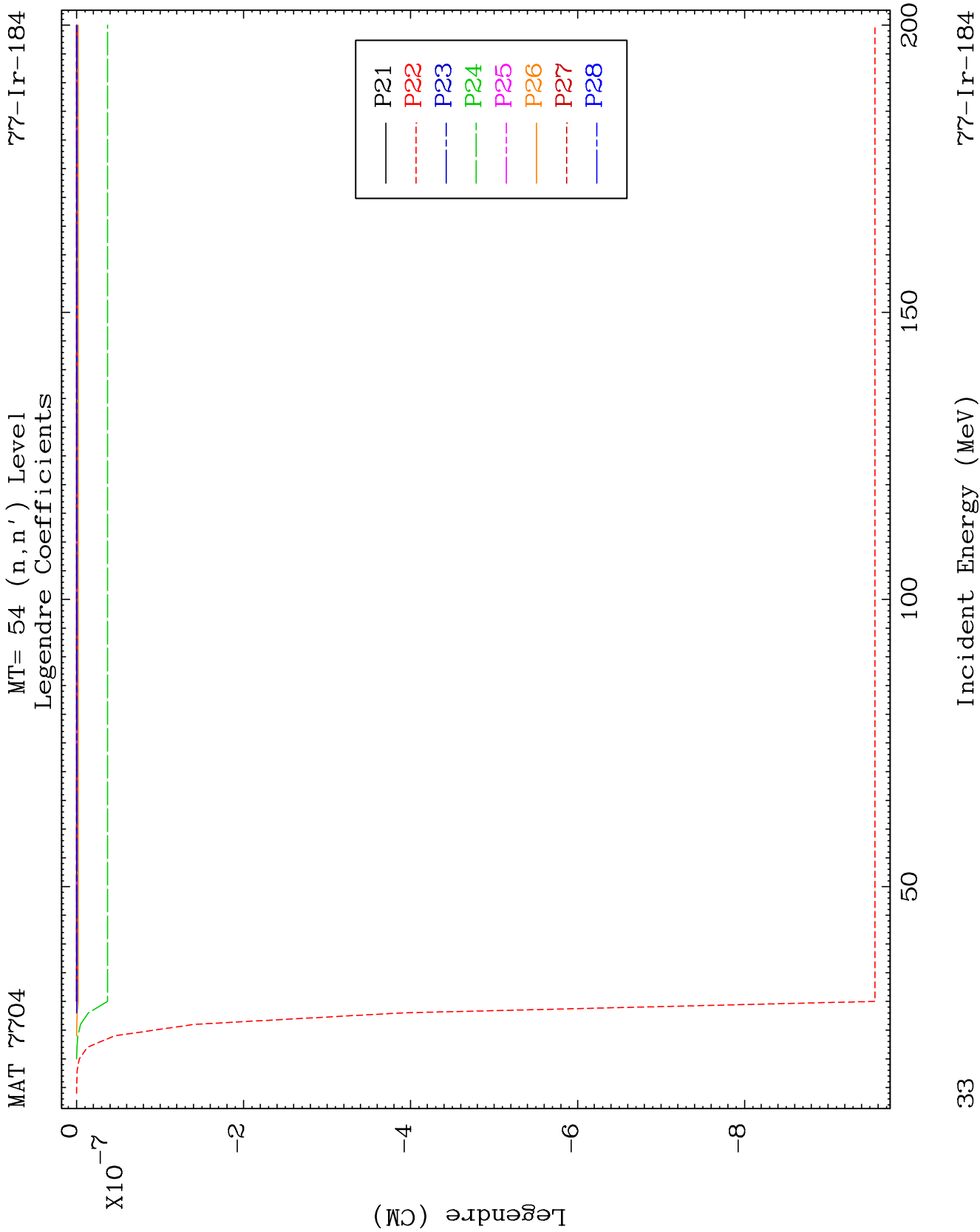


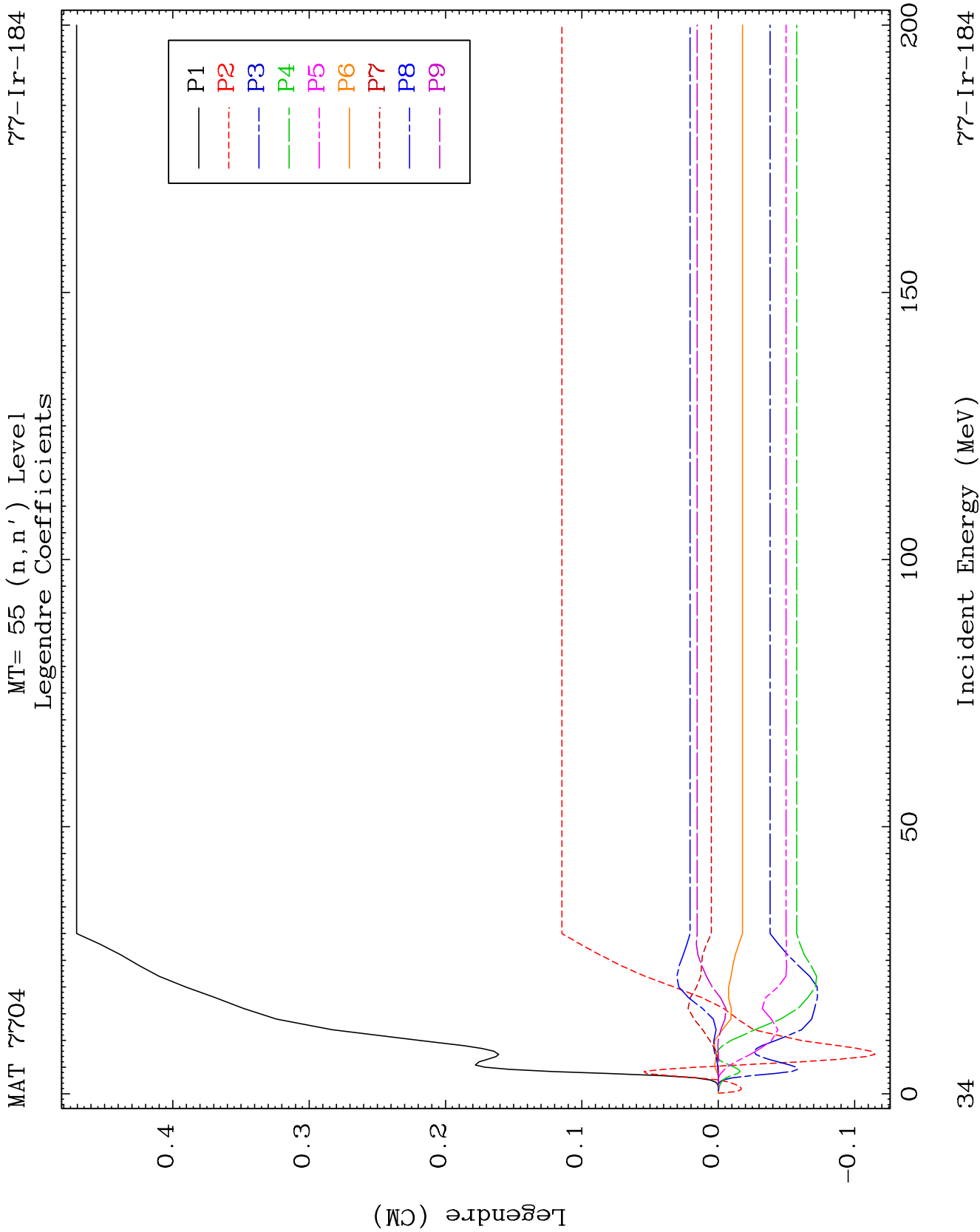








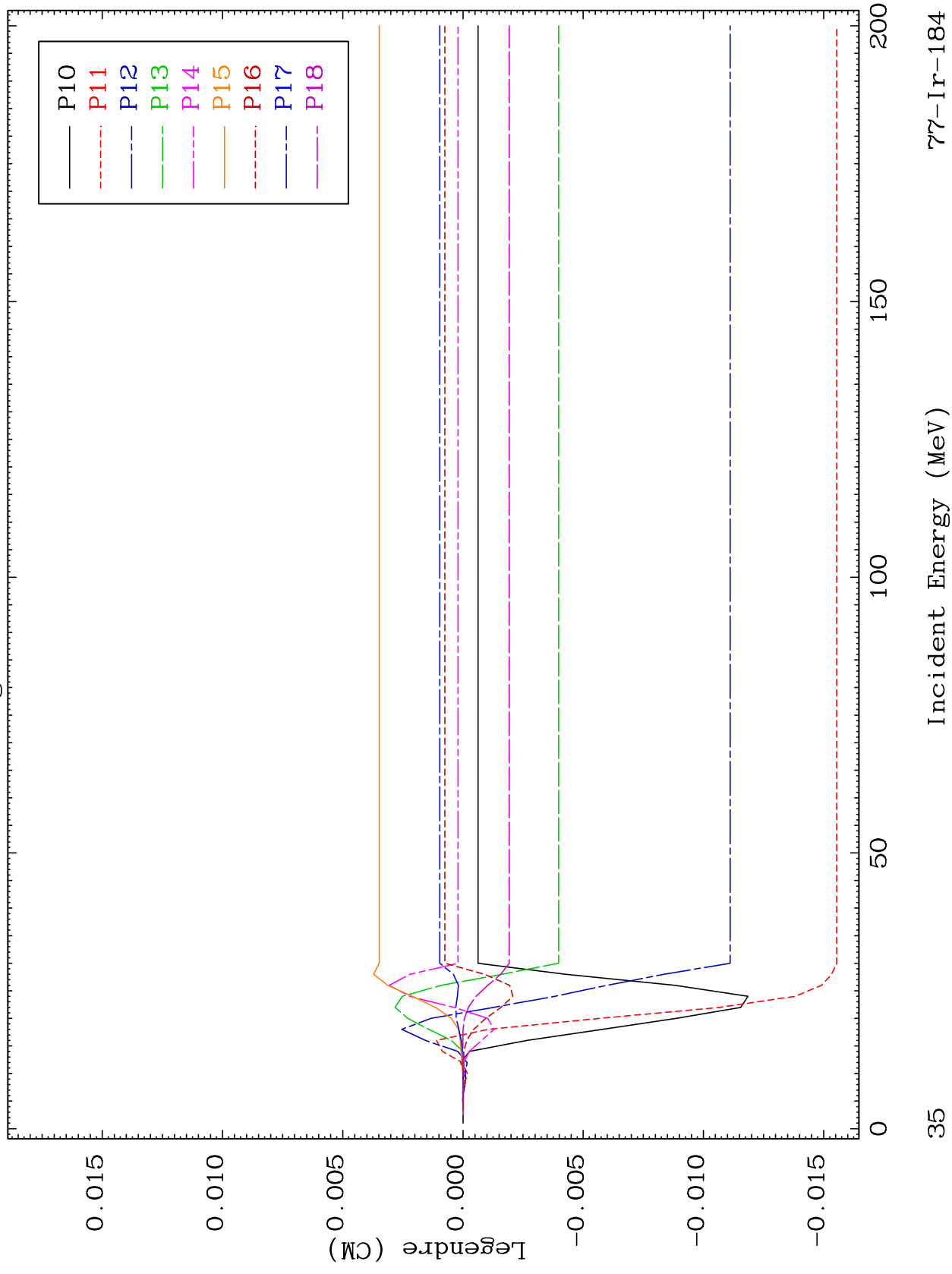


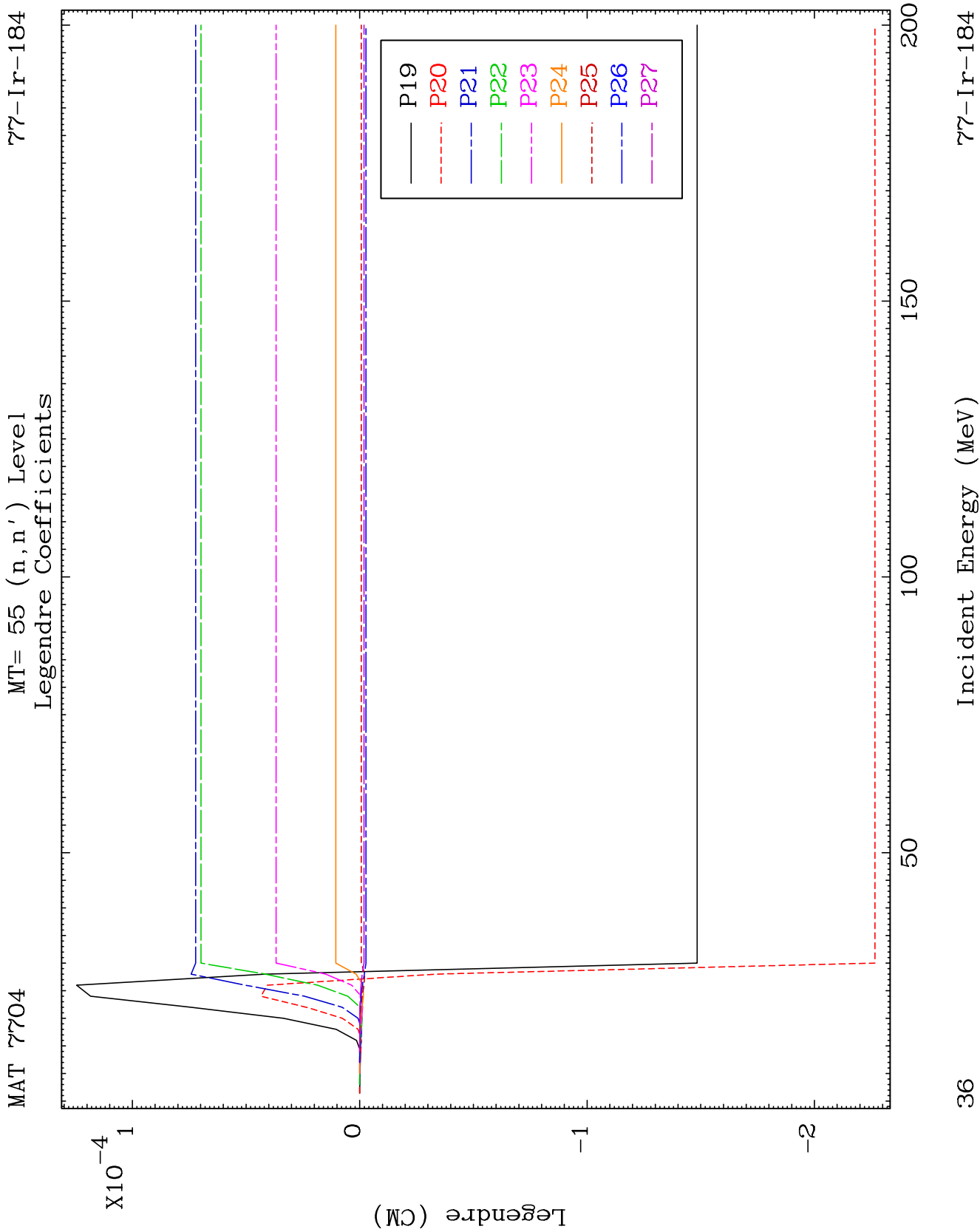


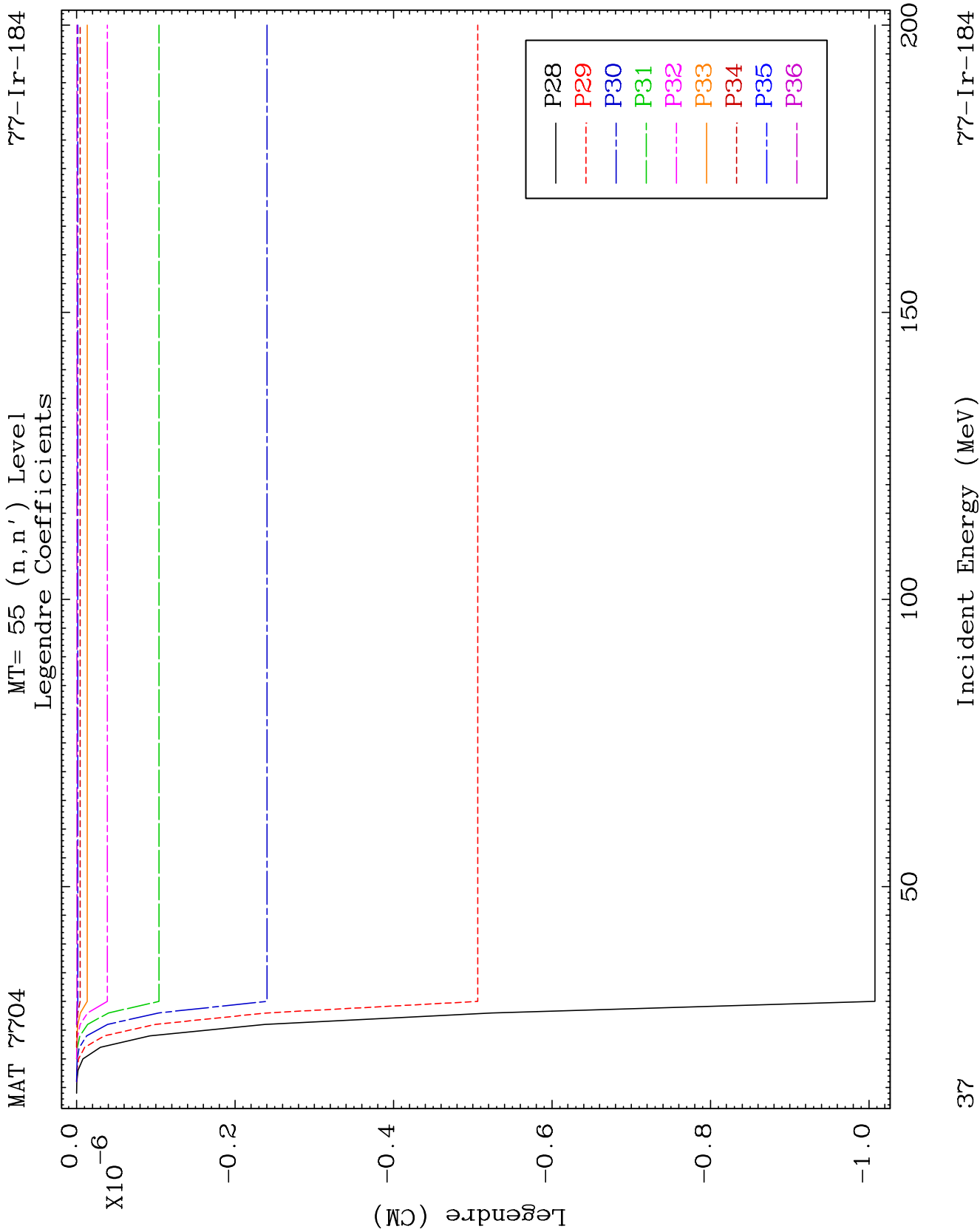
MAT 2704

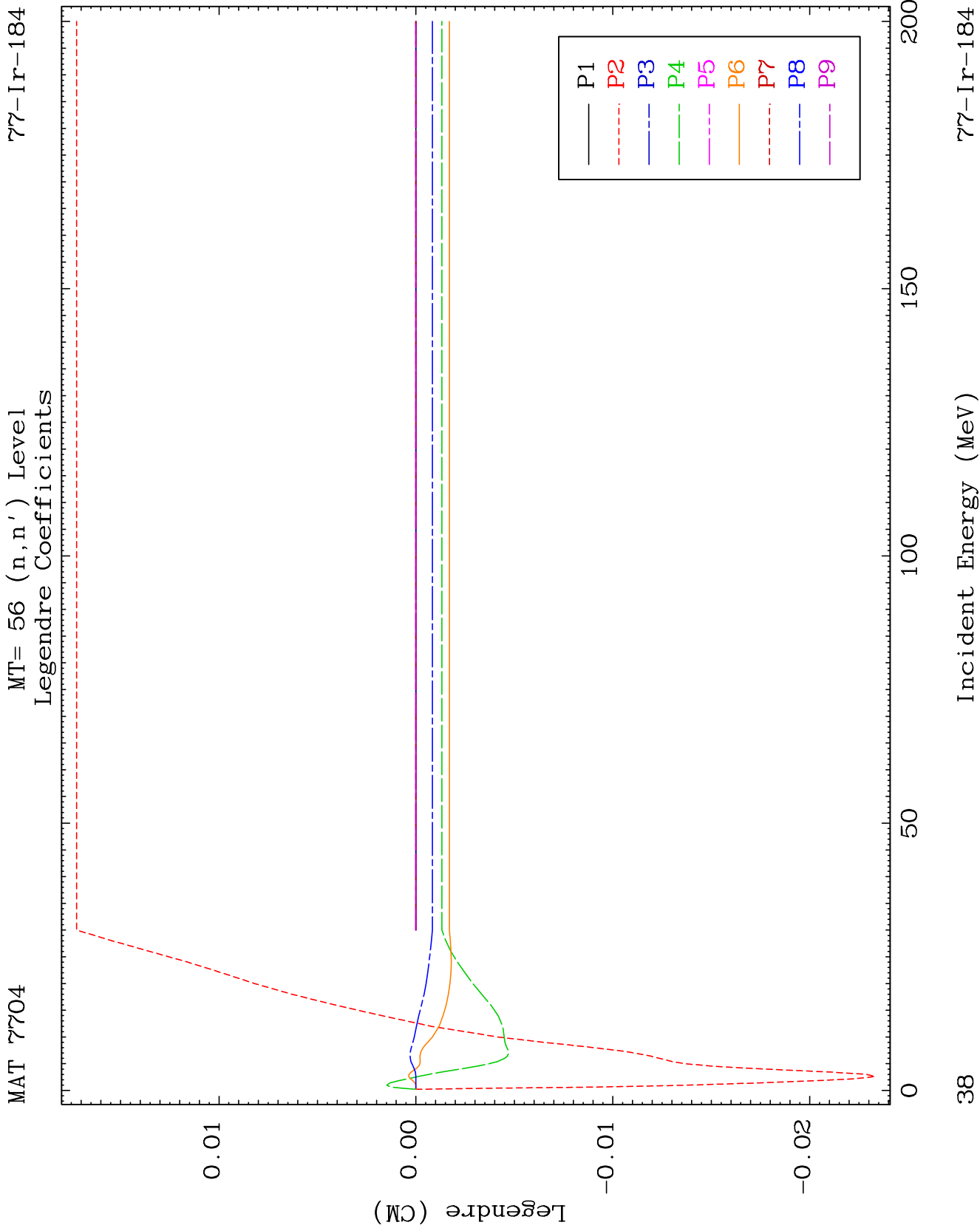
MT= 55 (n, n') Level
Legendre Coefficients

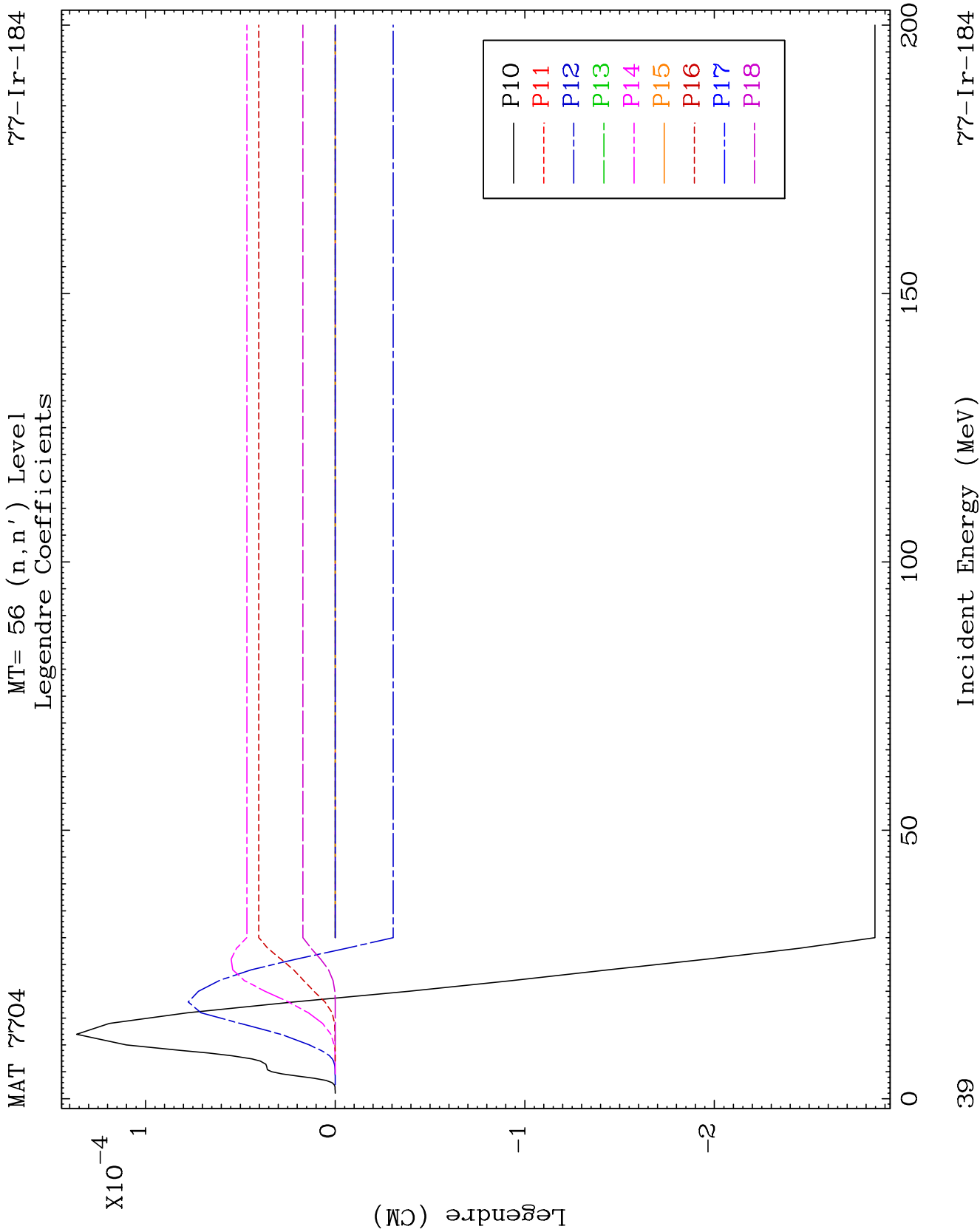
77-Ir-184

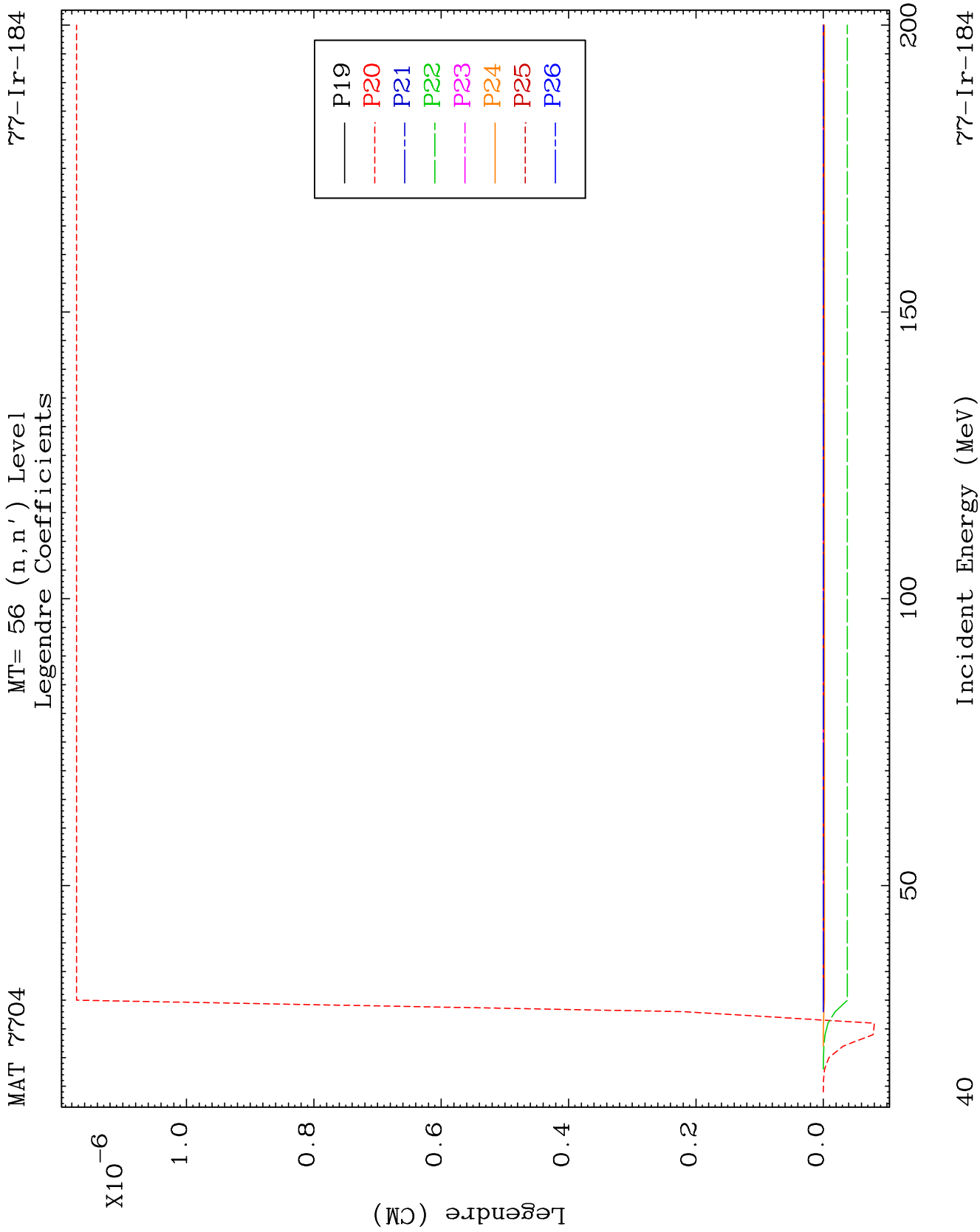


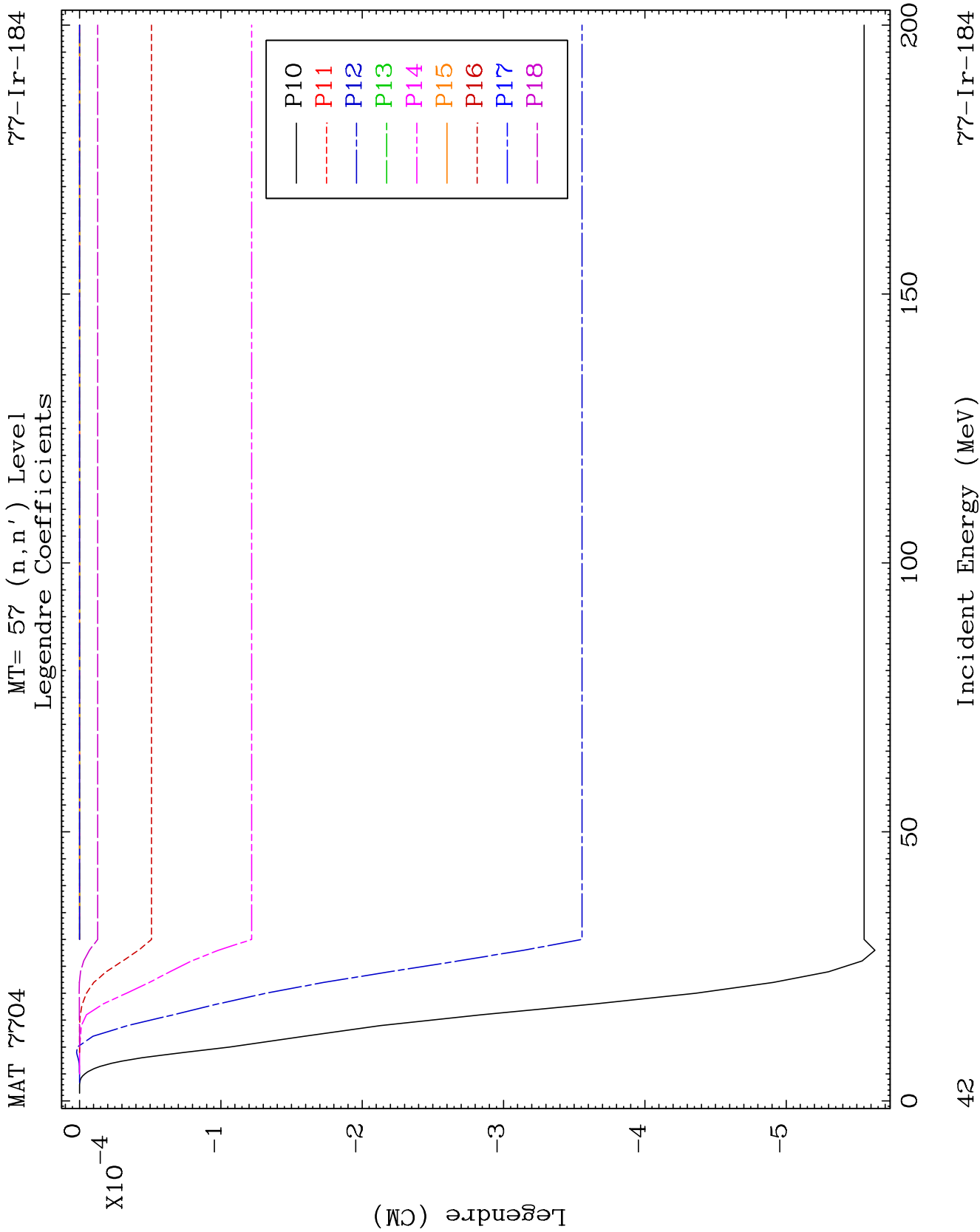








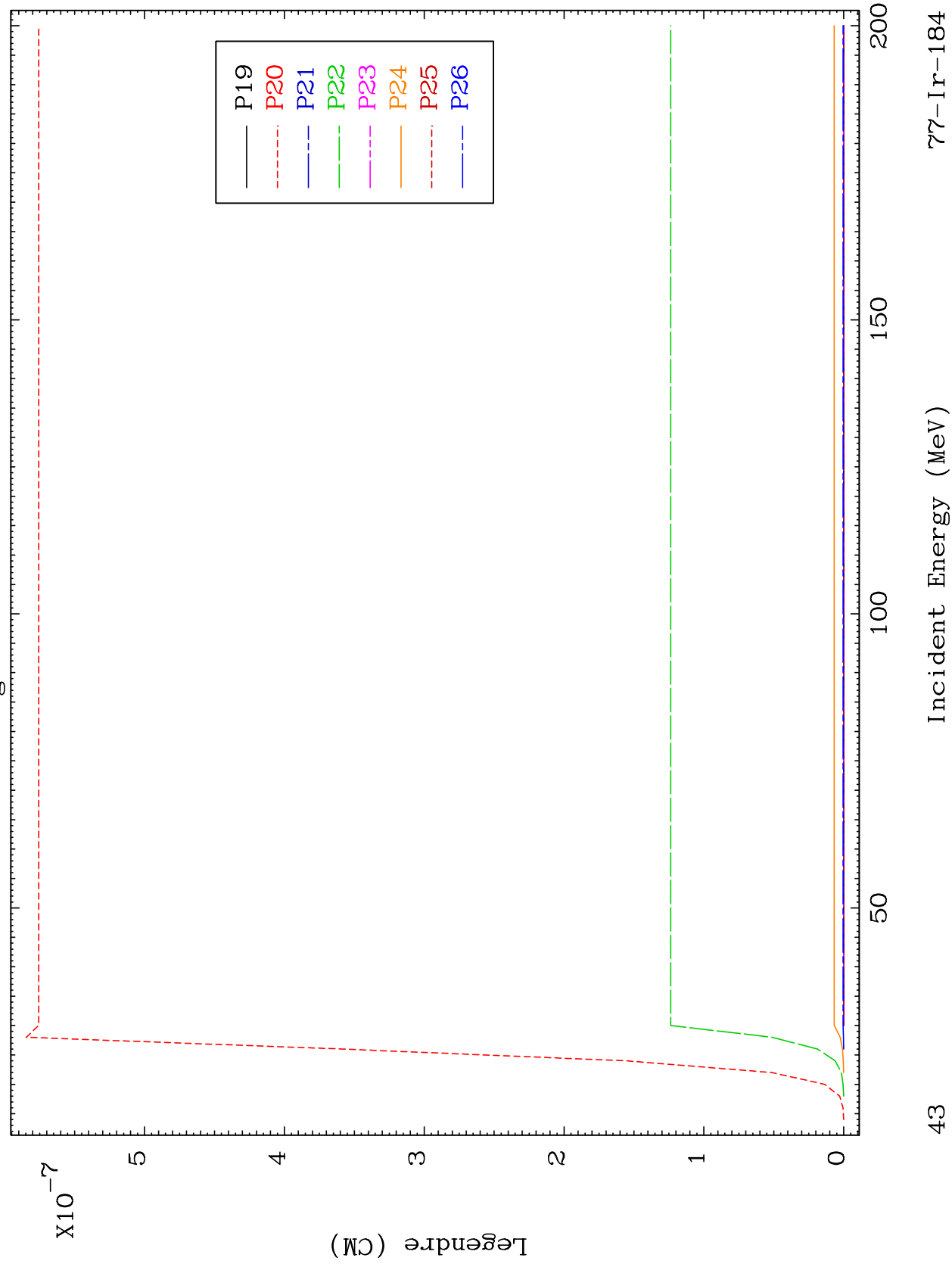


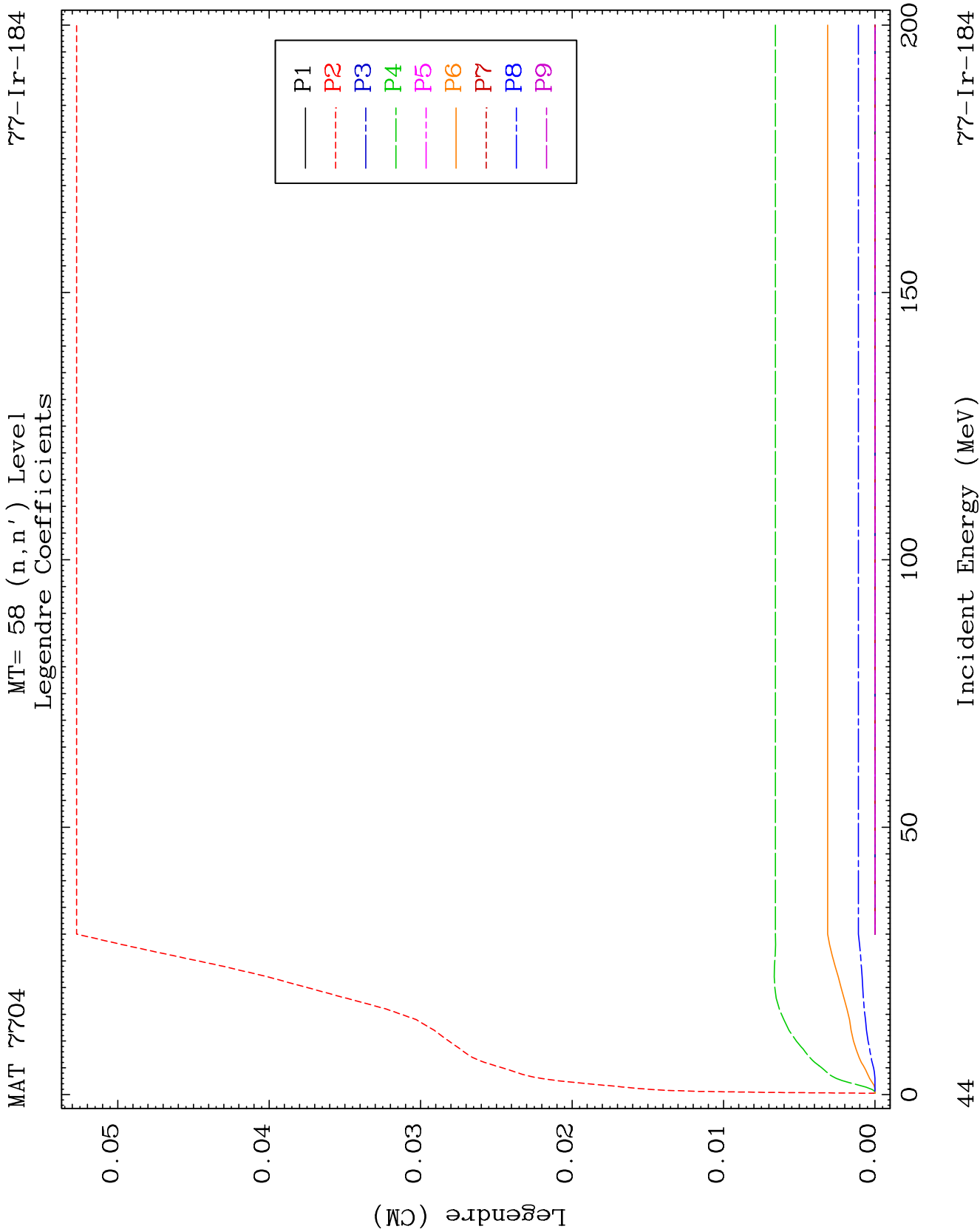


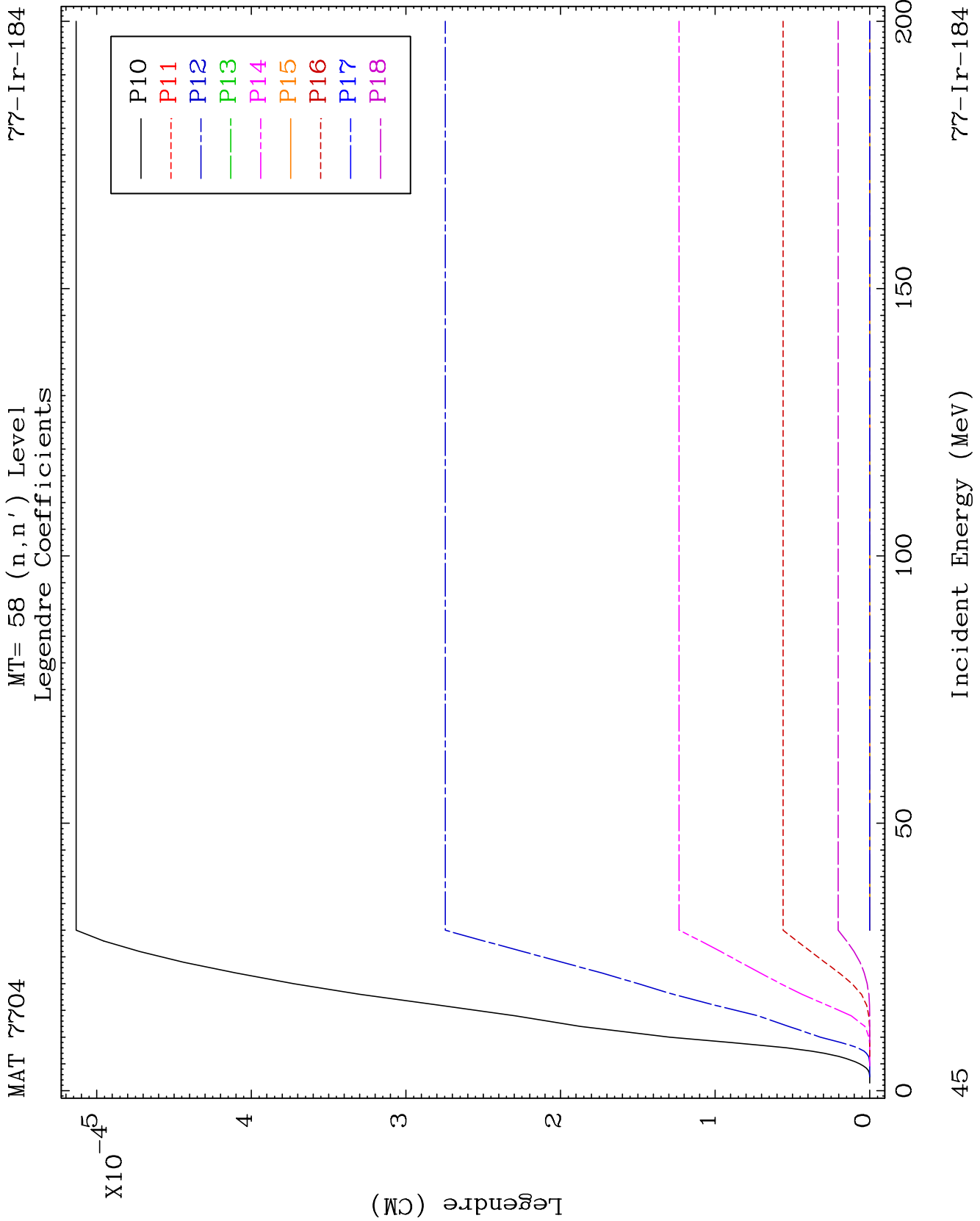
MAT 2704

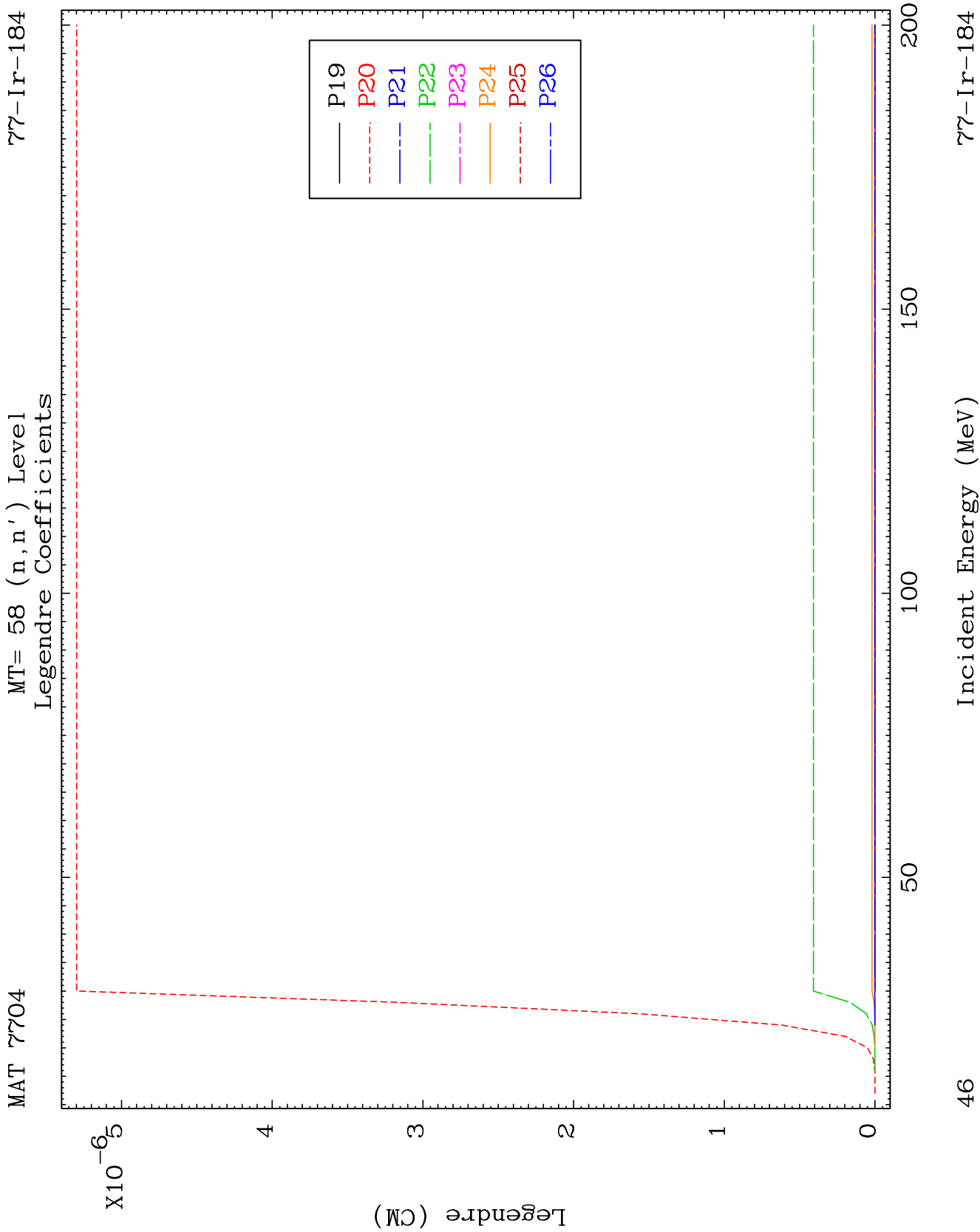
MT= 57 (n, n') Level
Legendre Coefficients

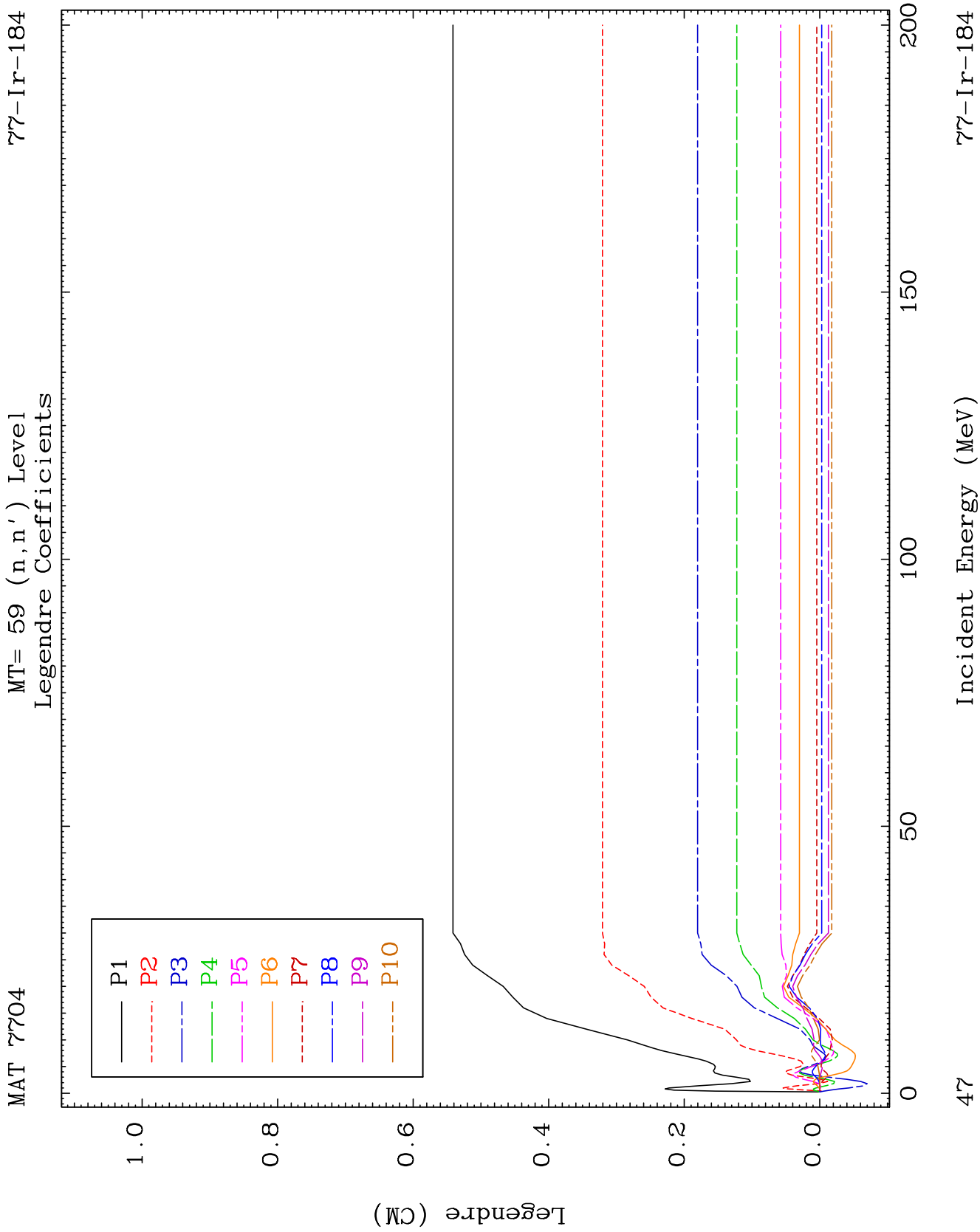
77-Ir-184







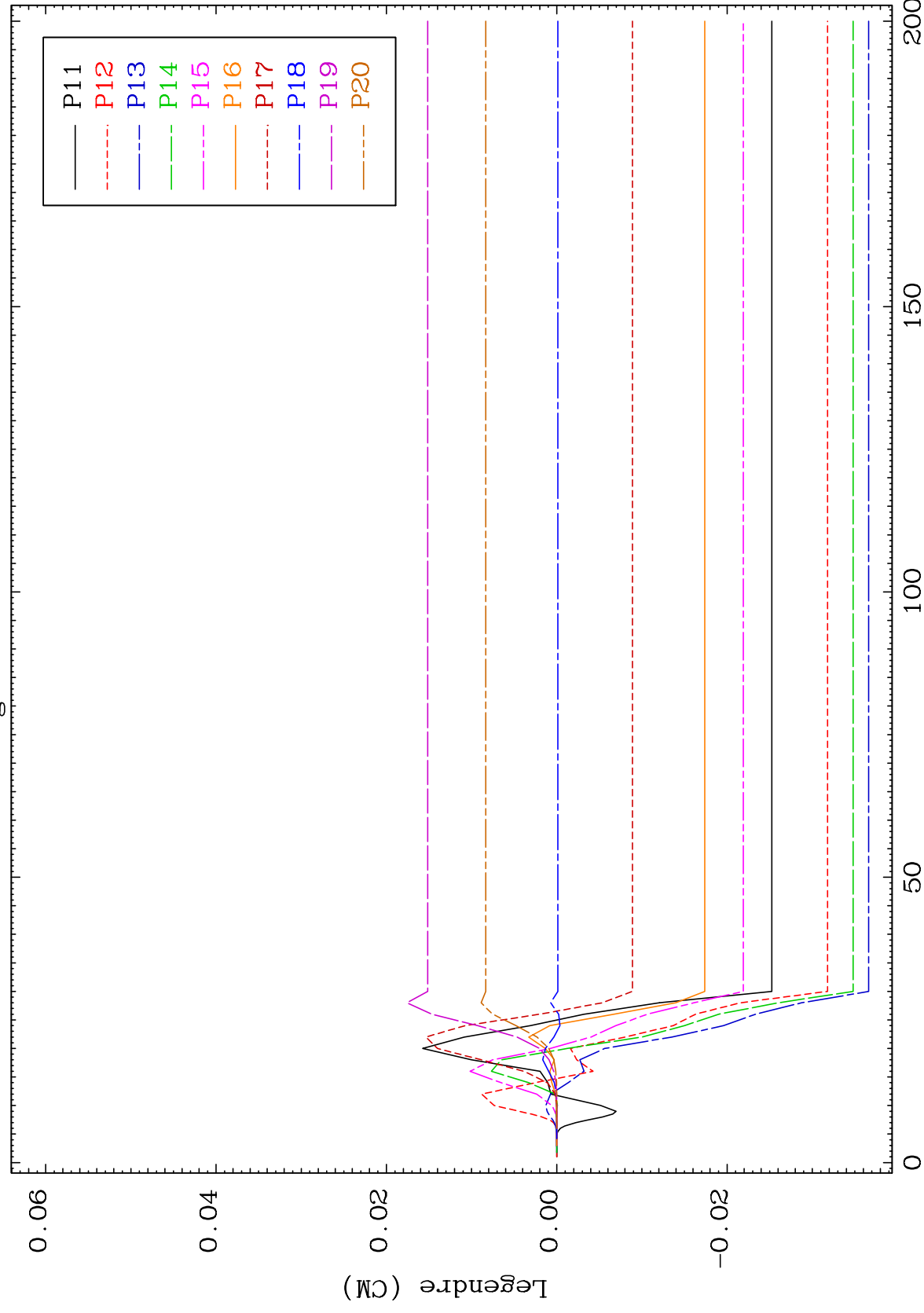




MAT 2704

MT= 59 (n, n') Level
Legendre Coefficients

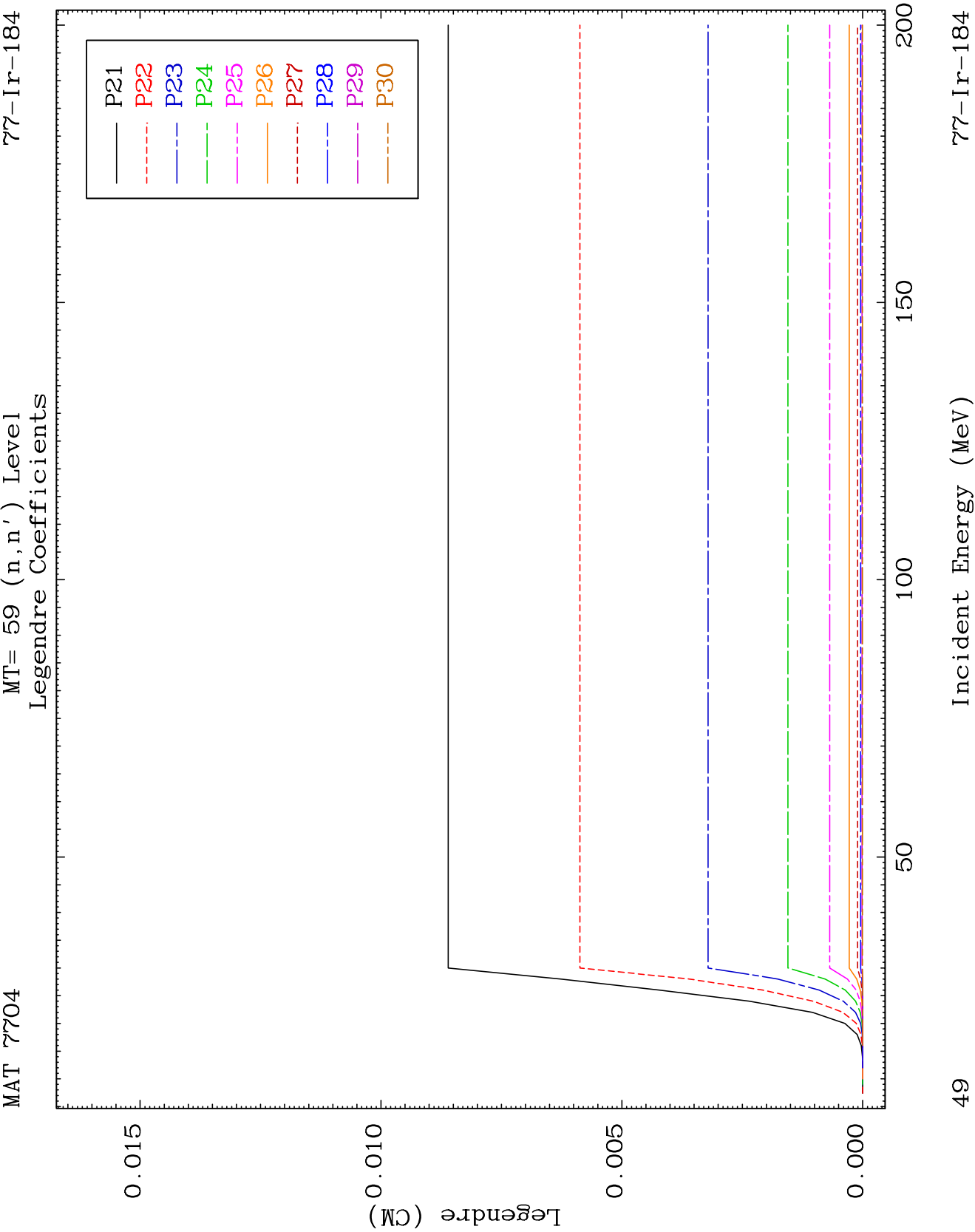
77-Ir-184

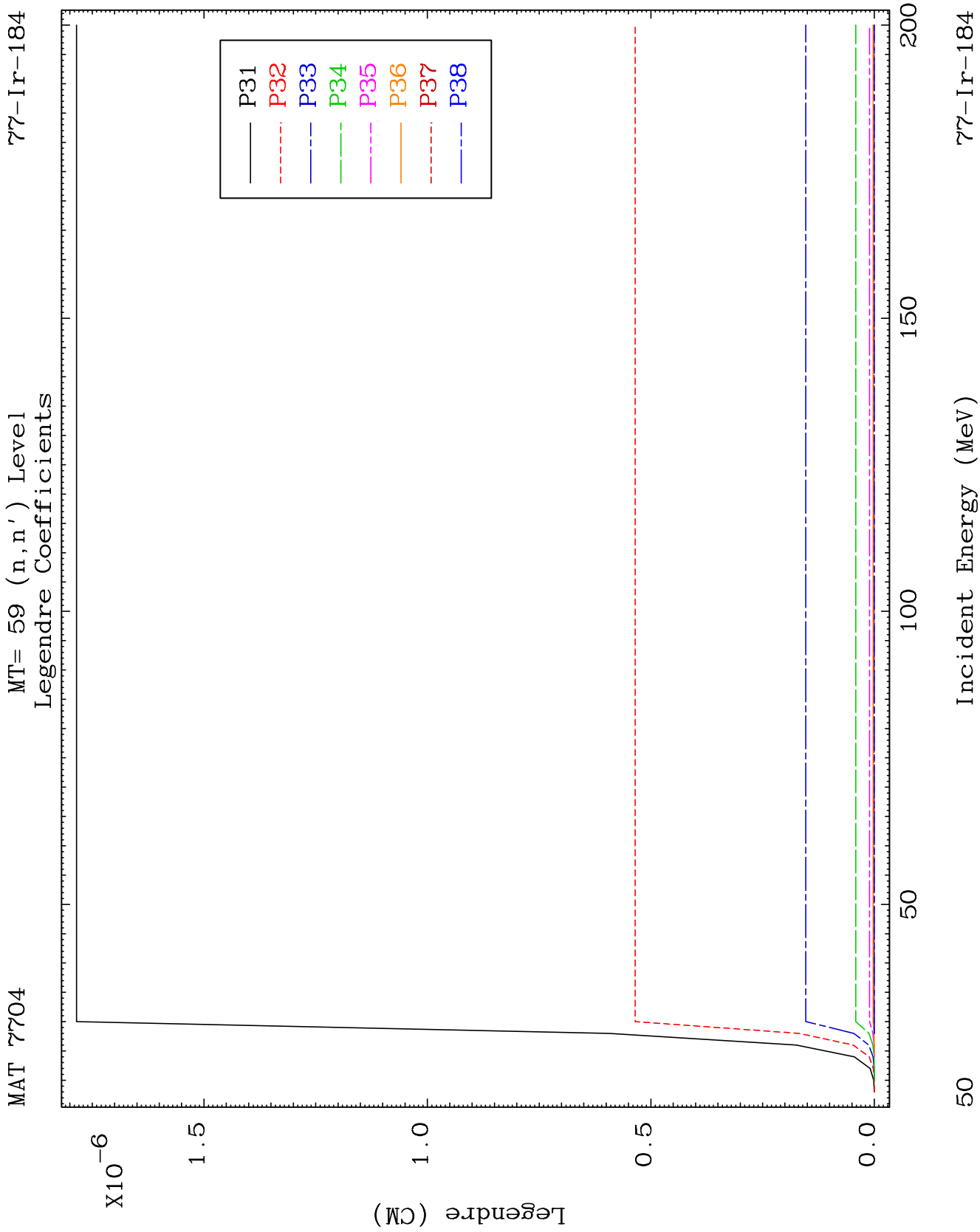


48

Incident Energy (MeV)

77-Ir-184

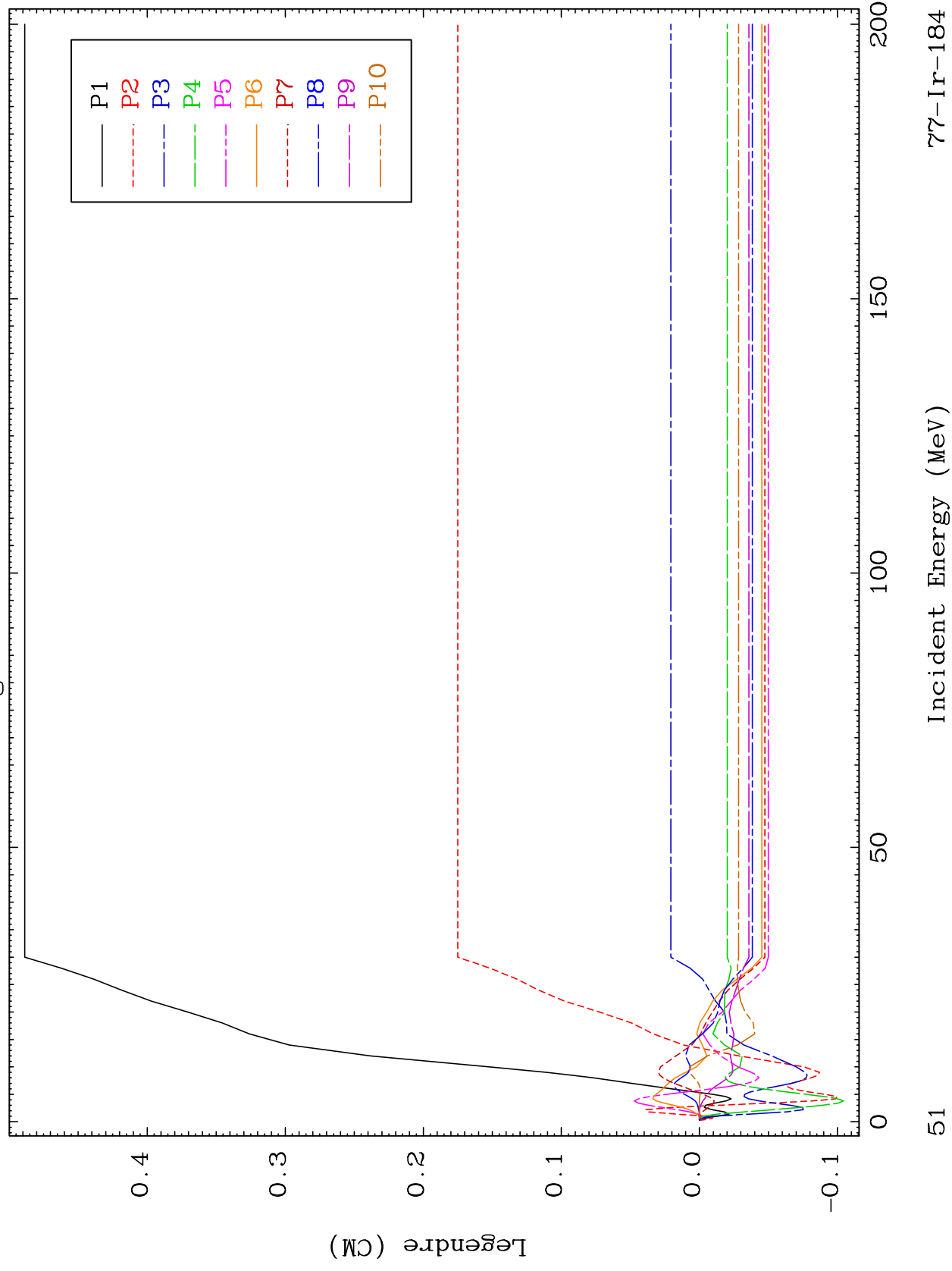




MAT 2704

MT= 60 (n, n') Level
Legendre Coefficients

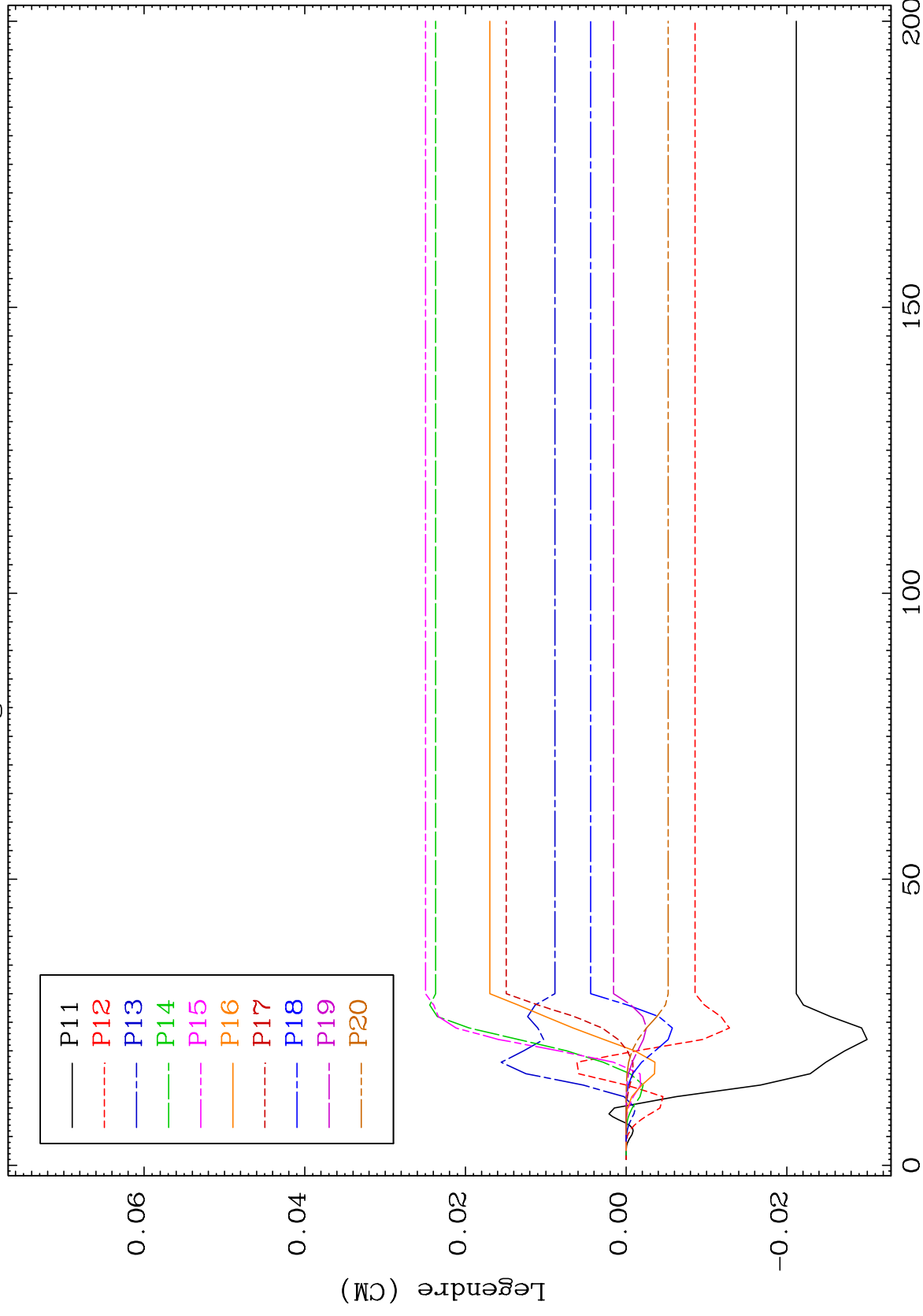
77-Ir-184



MAT 2704

MT= 60 (n, n') Level
Legendre Coefficients

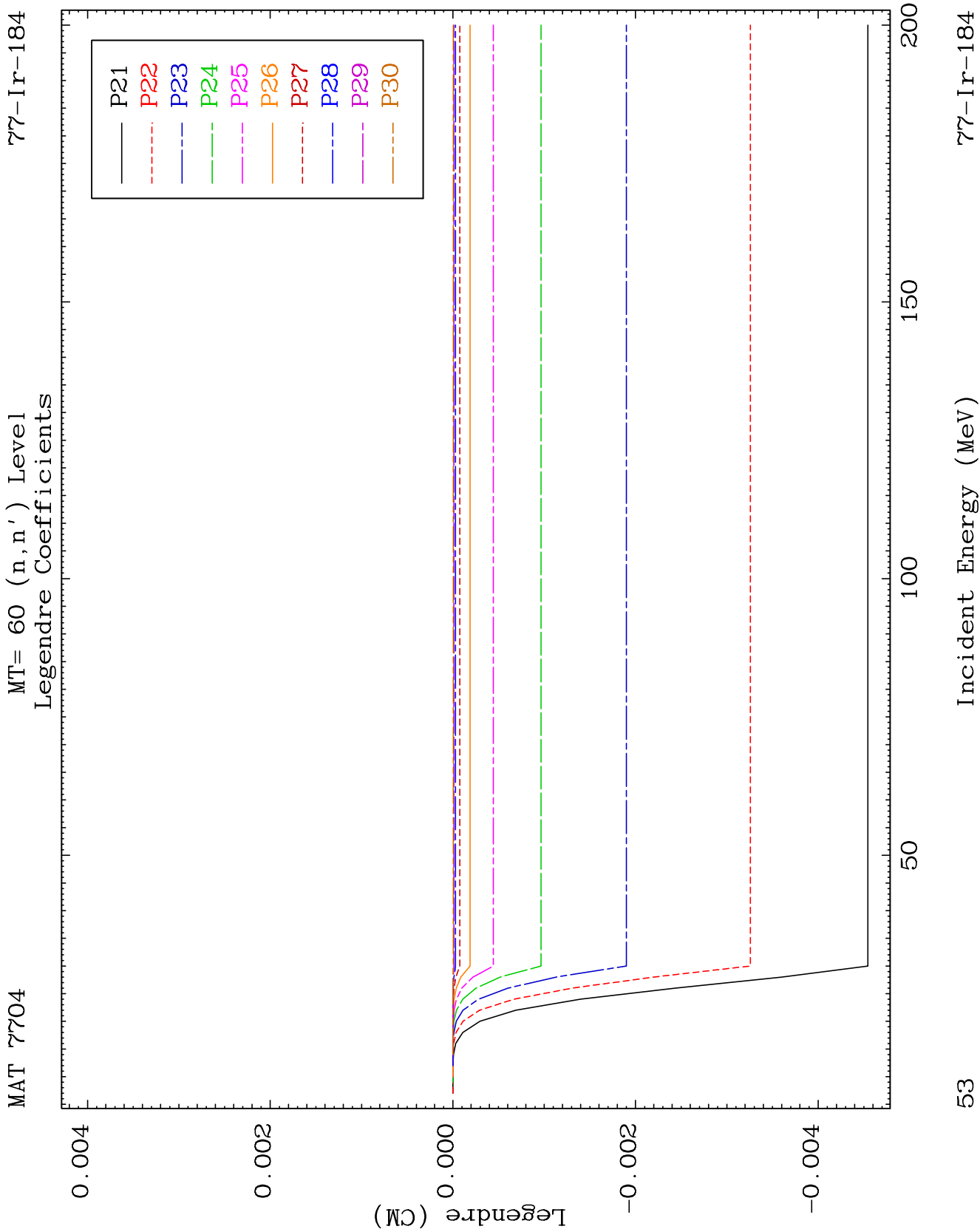
77-Ir-184

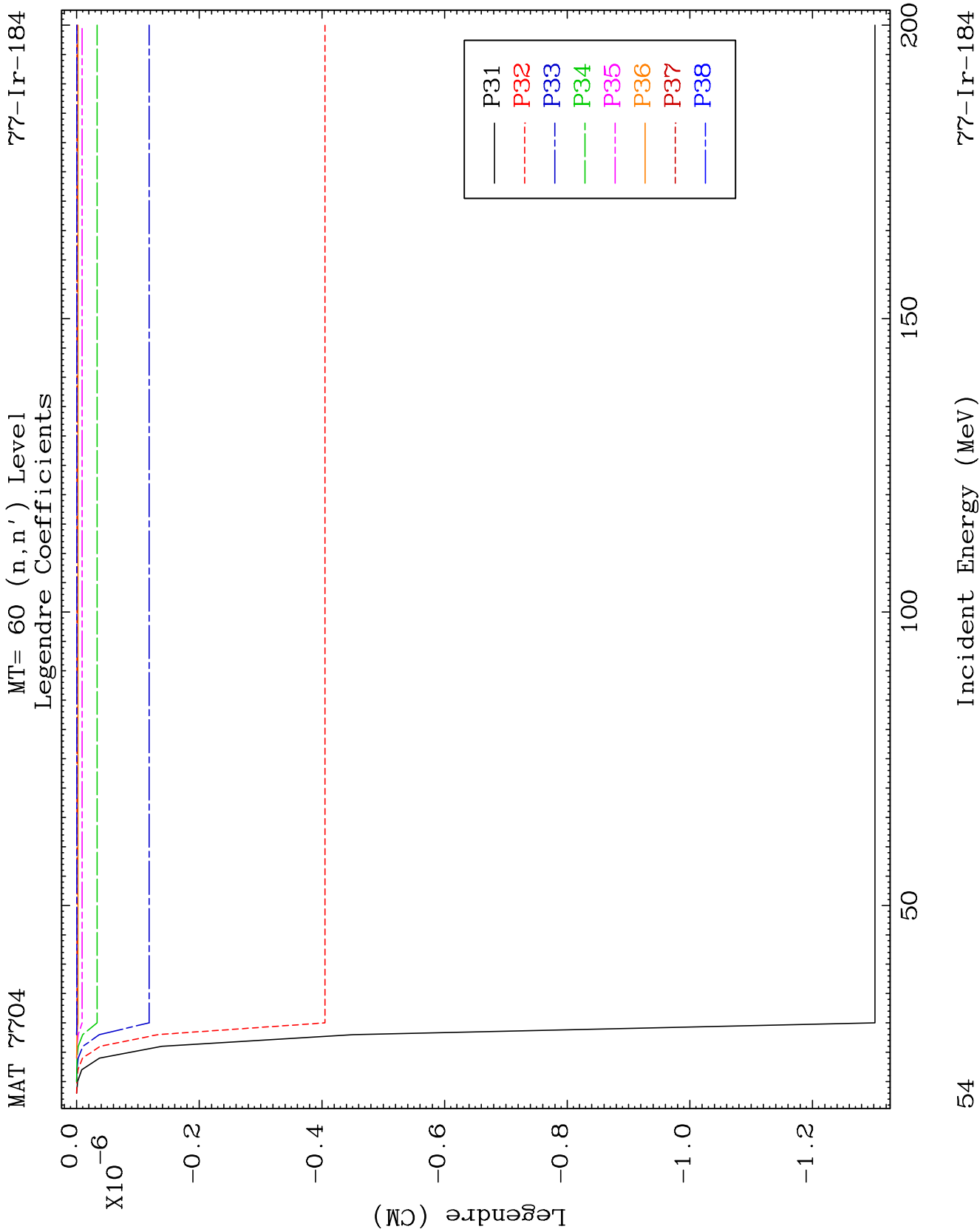


52

Incident Energy (MeV)

77-Ir-184

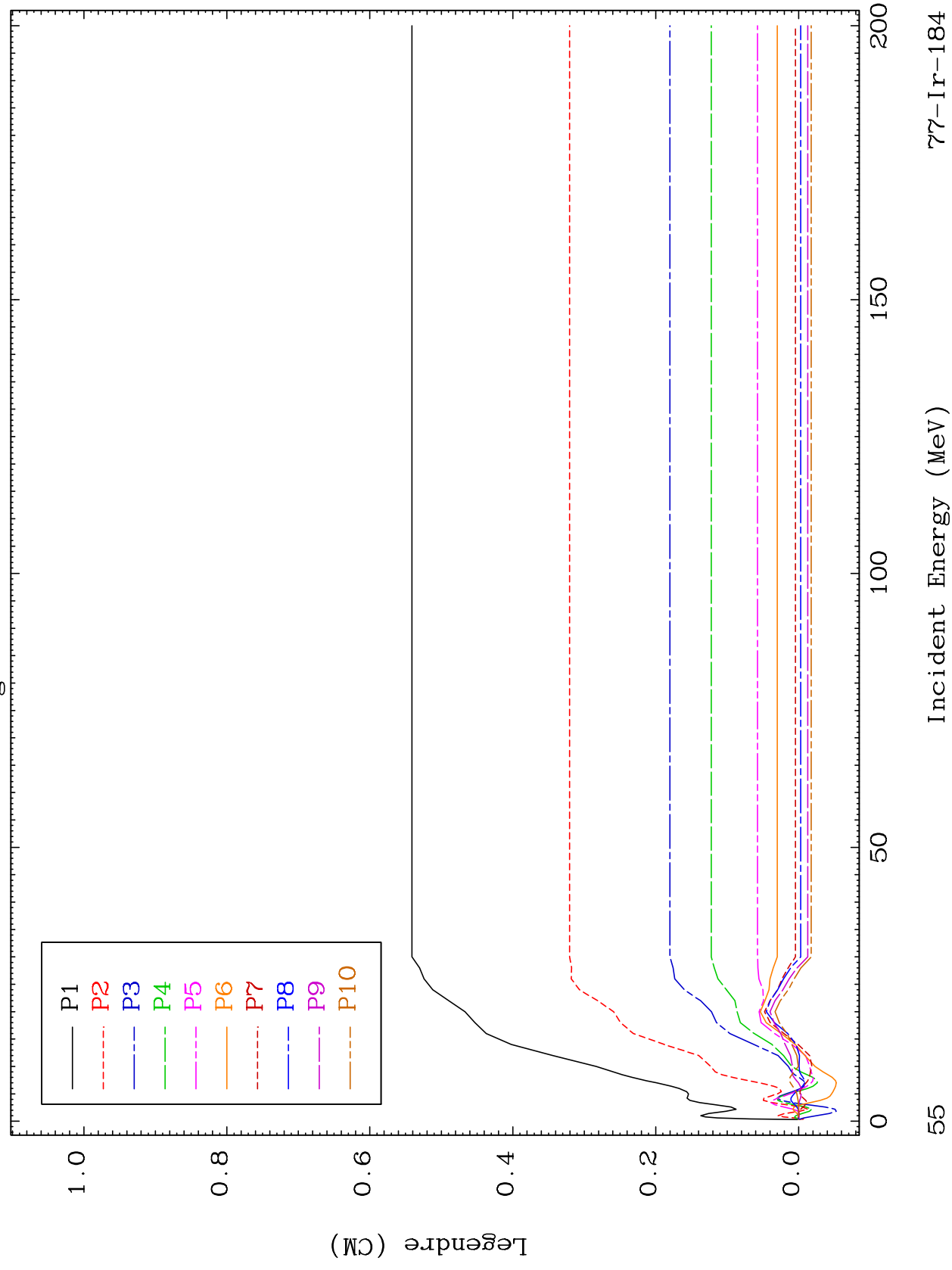




MAT 2704

MT= 61 (n, n') Level
Legendre Coefficients

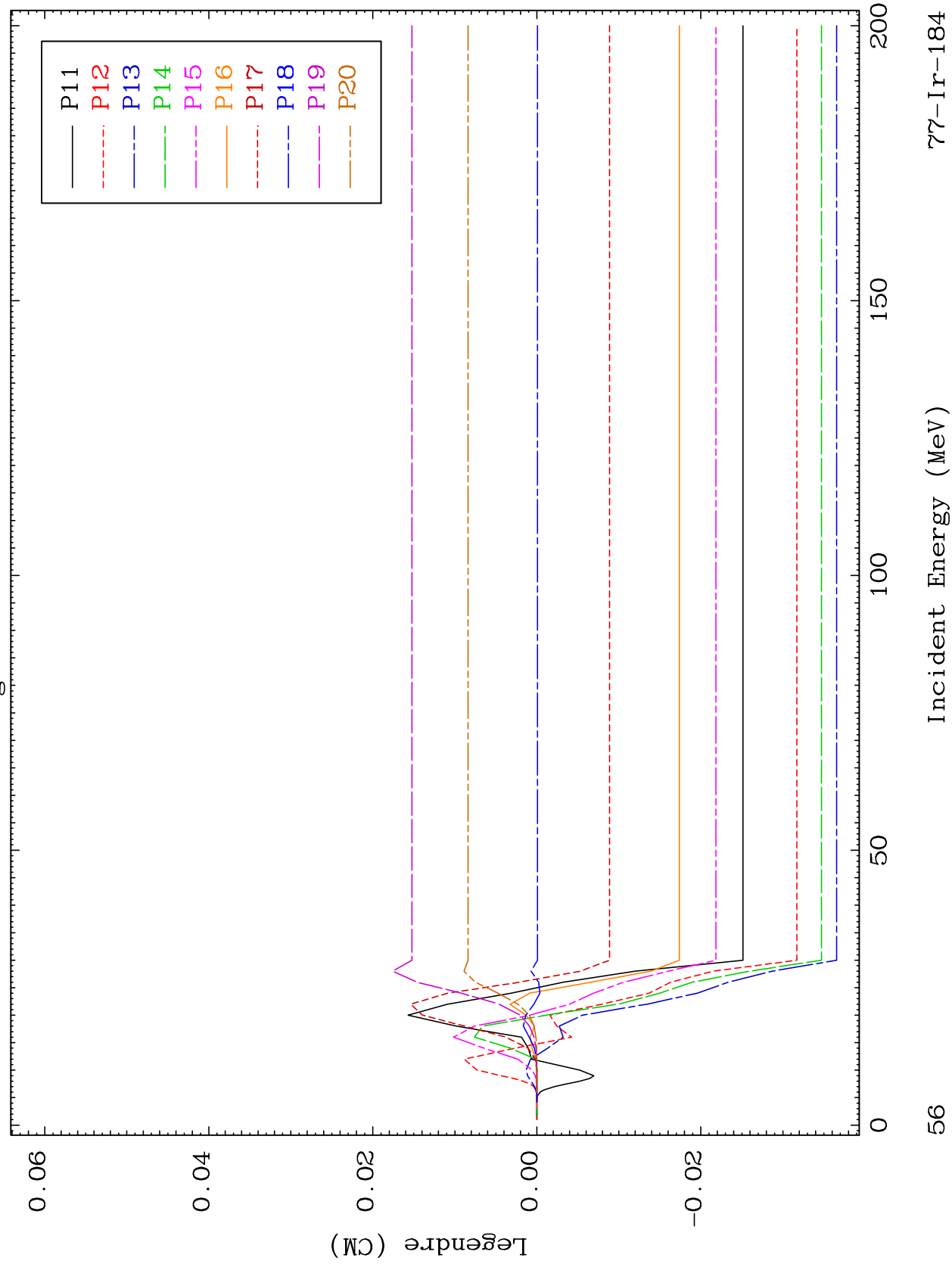
77-Ir-184

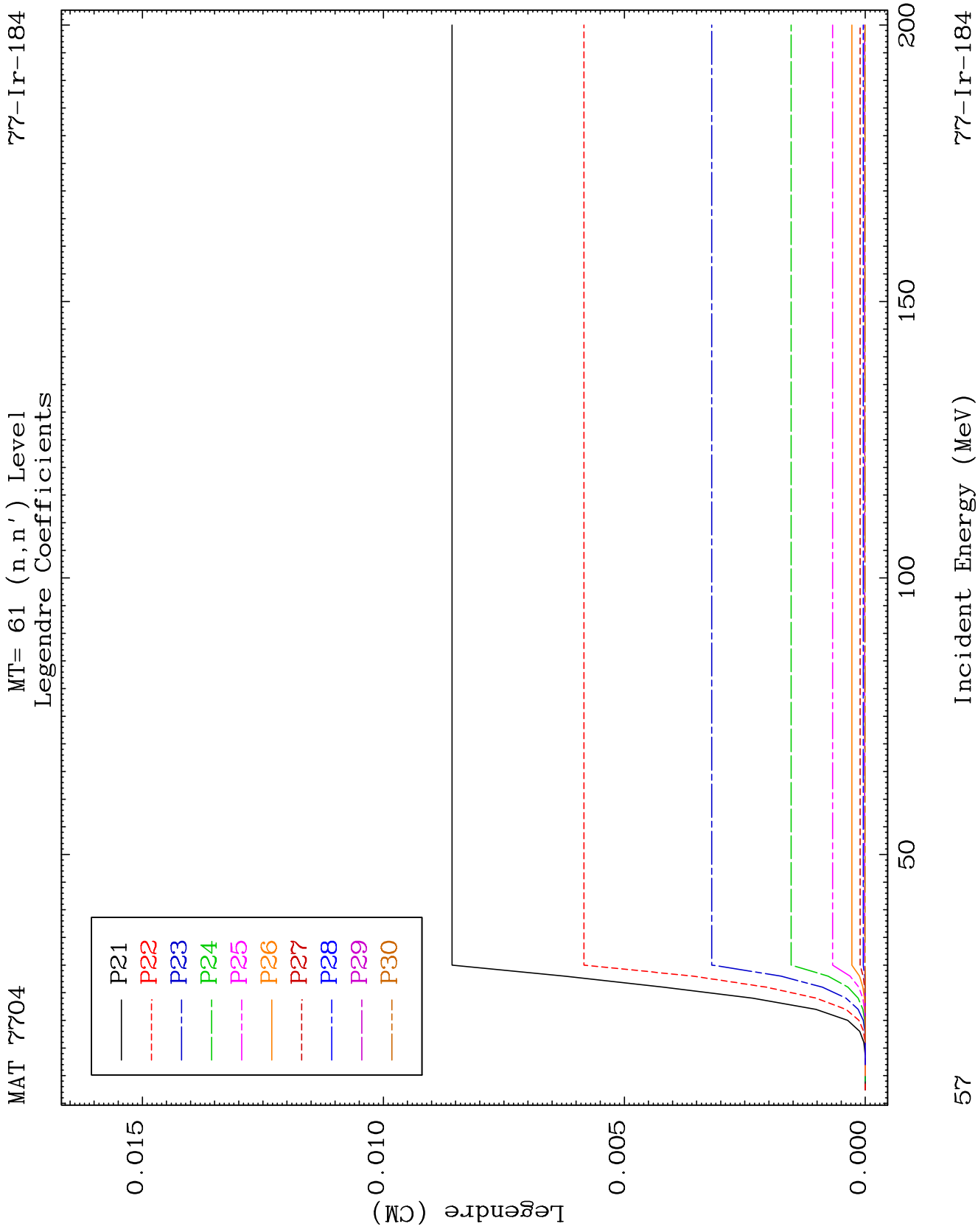


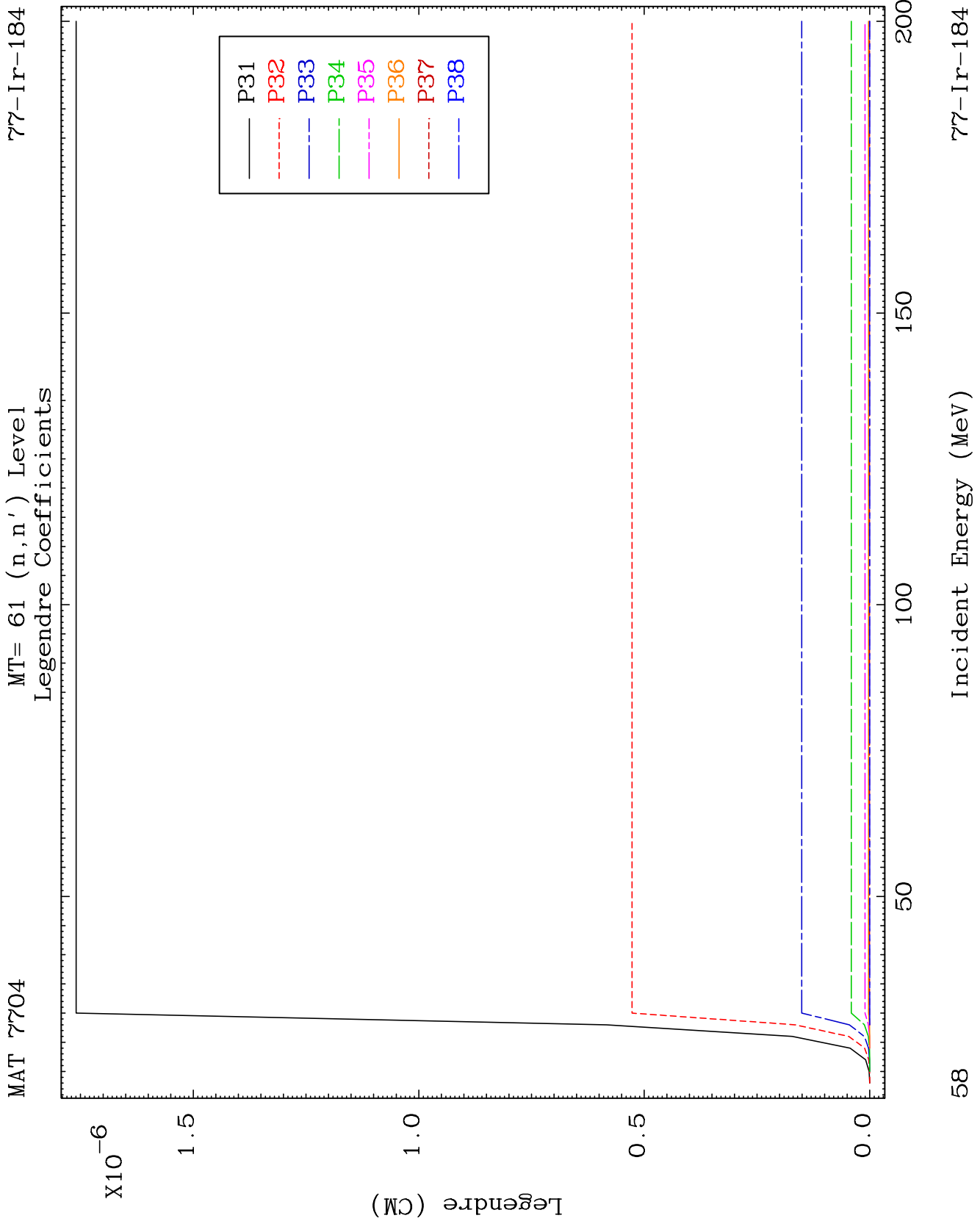
MAT 2704

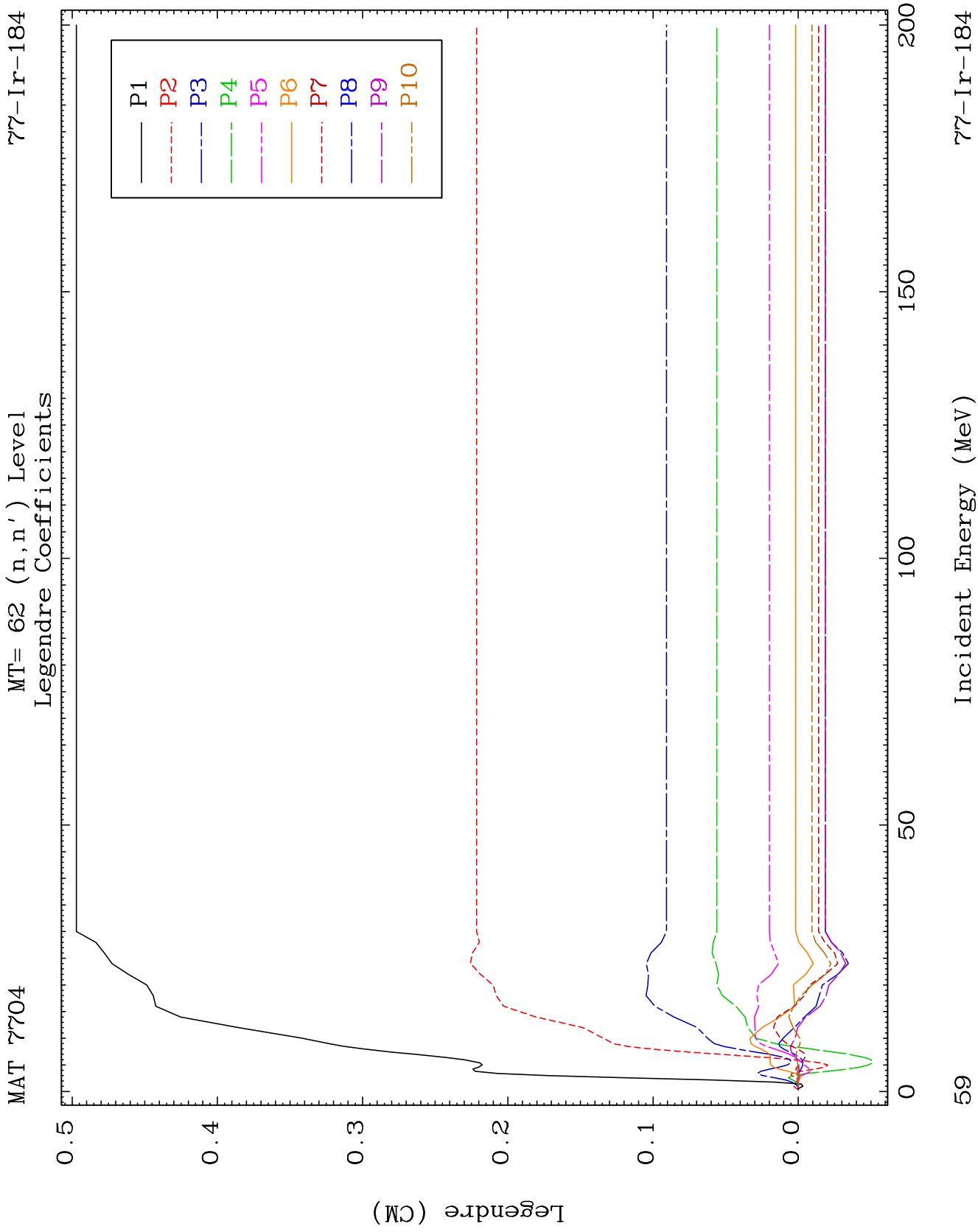
MT= 61 (n, n') Level
Legendre Coefficients

77-Ir-184





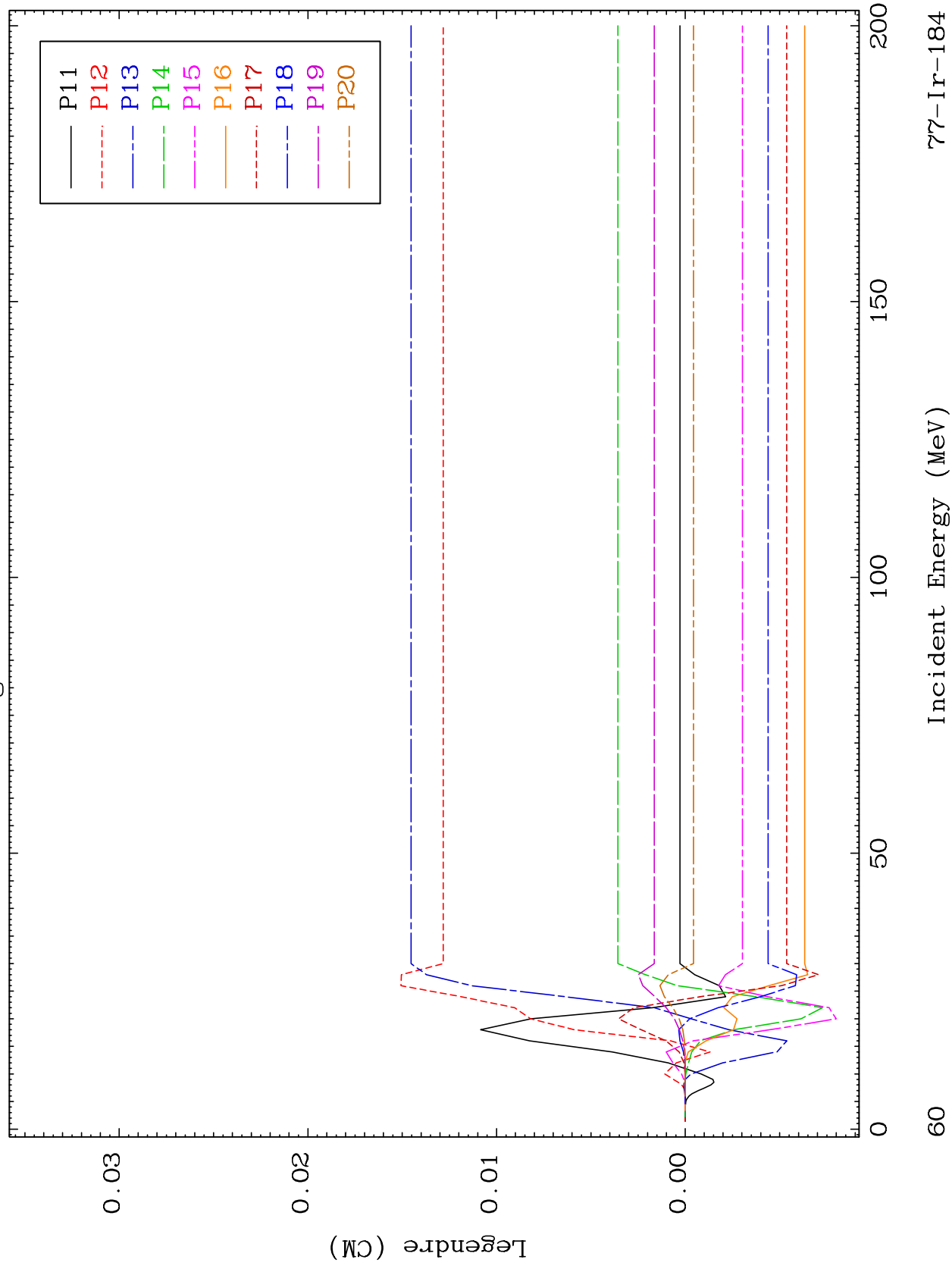




MAT 2704

MT= 62 (n, n') Level
Legendre Coefficients

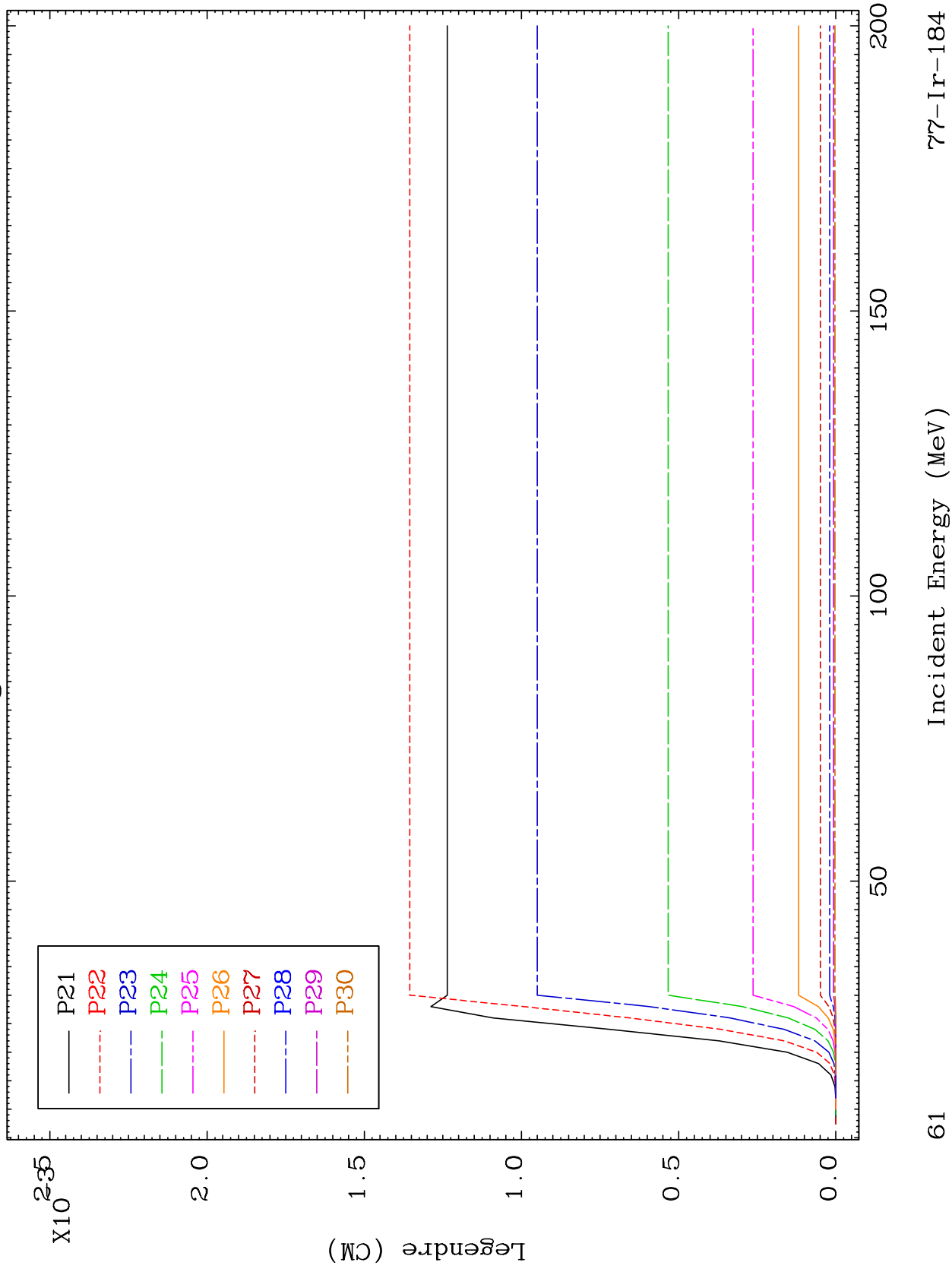
77-Ir-184

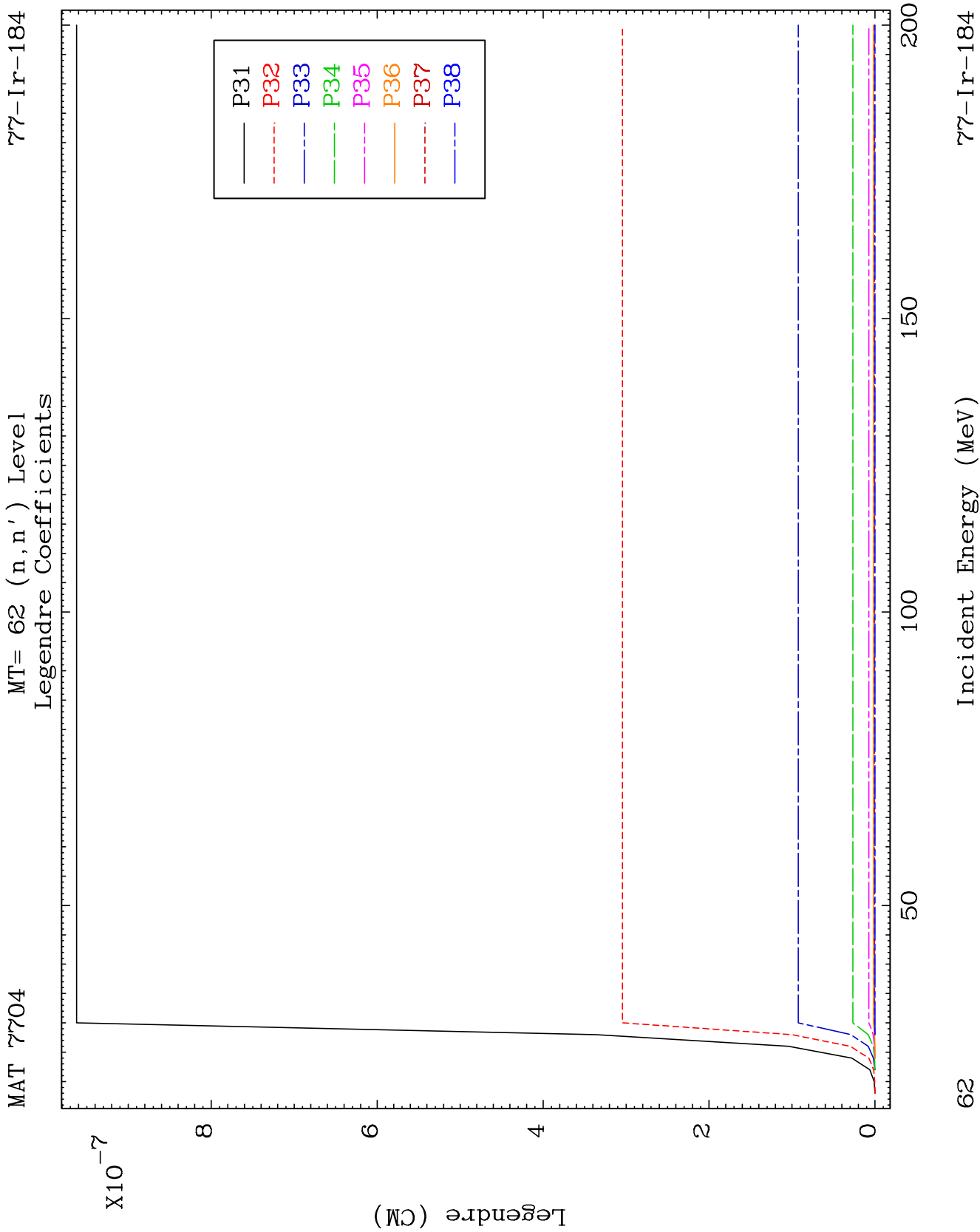


MAT 2704

MT= 62 (n, n') Level
Legendre Coefficients

77-Ir-184

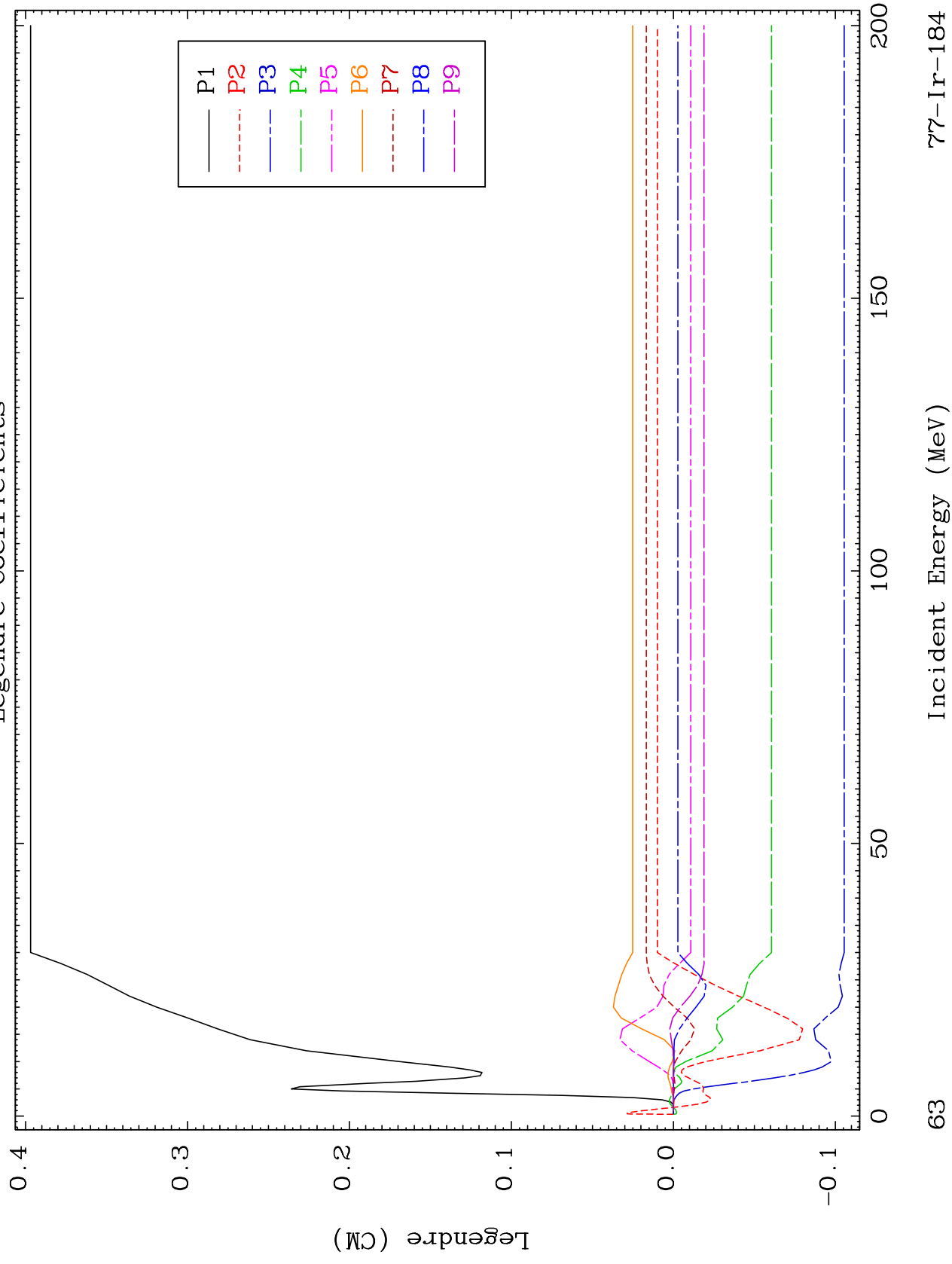


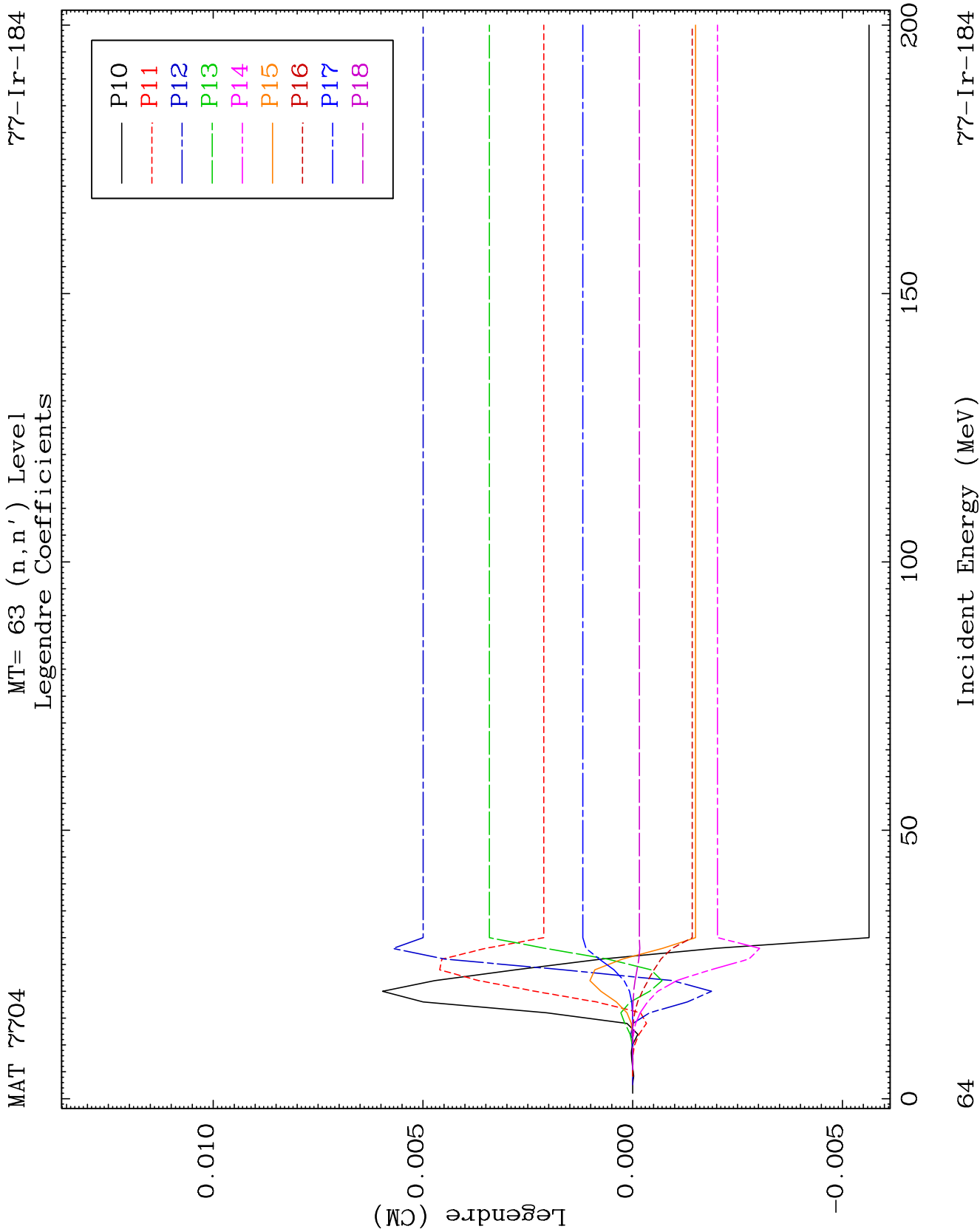


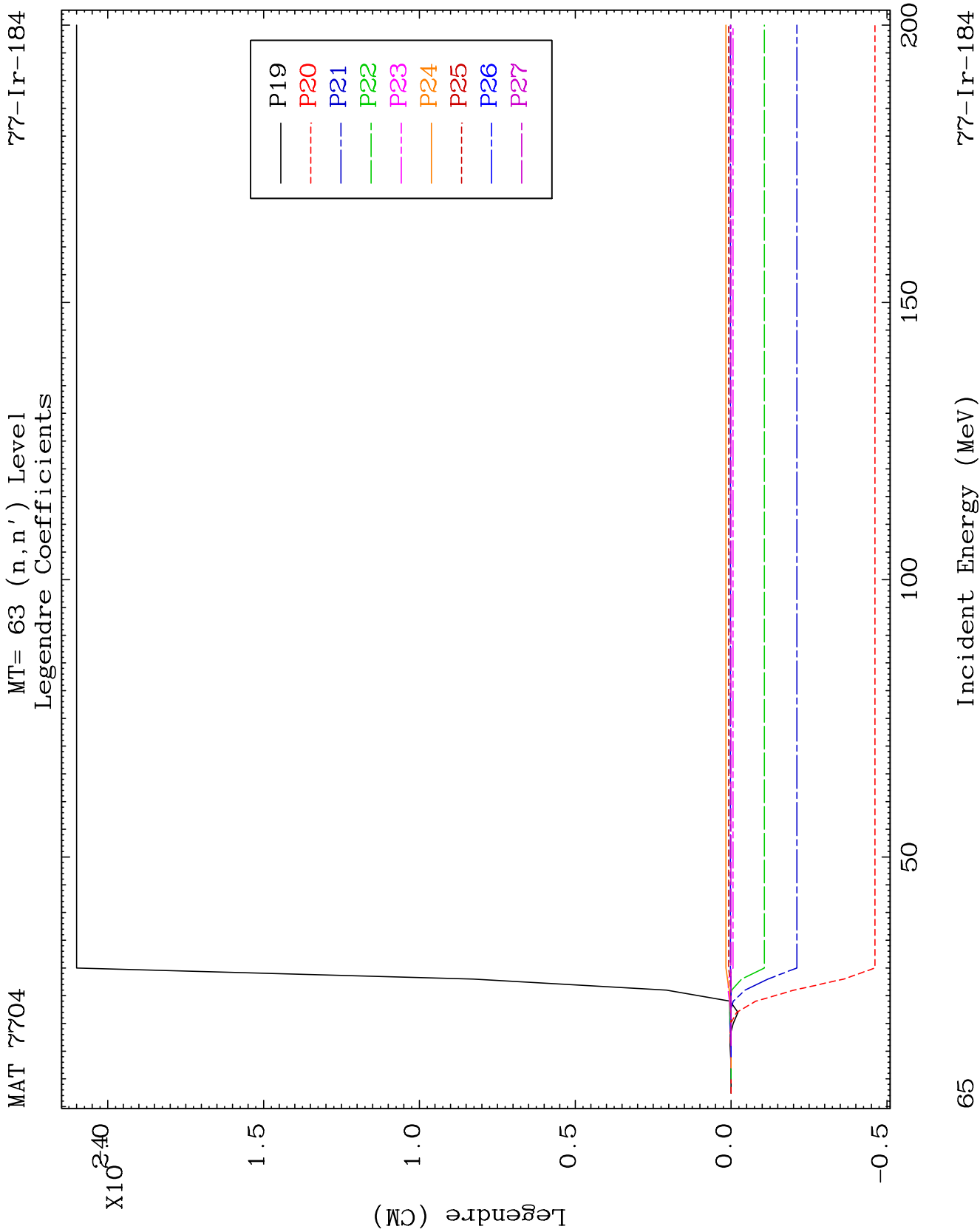
MAT 7704

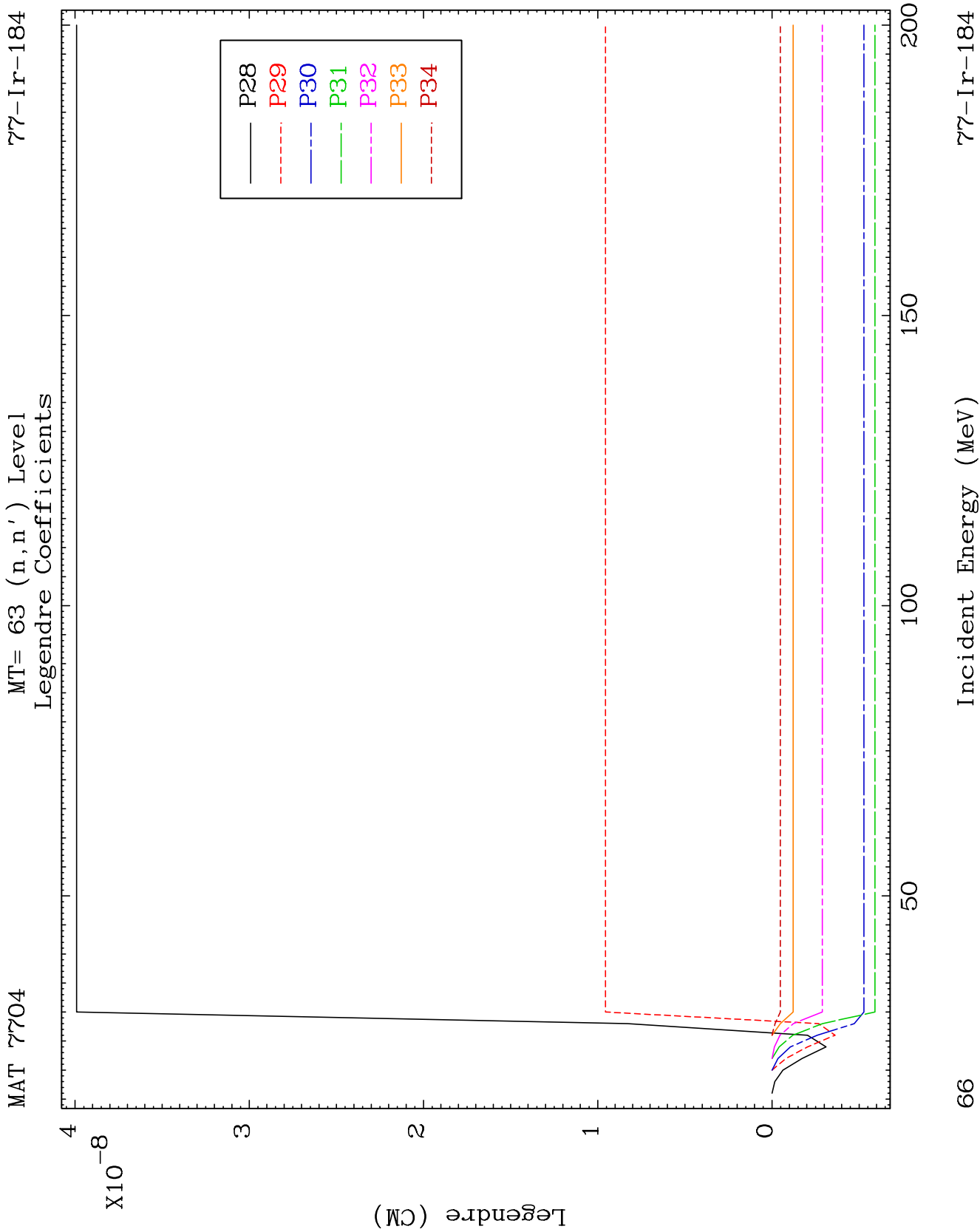
MT= 63 (n, n') Level
Legendre Coefficients

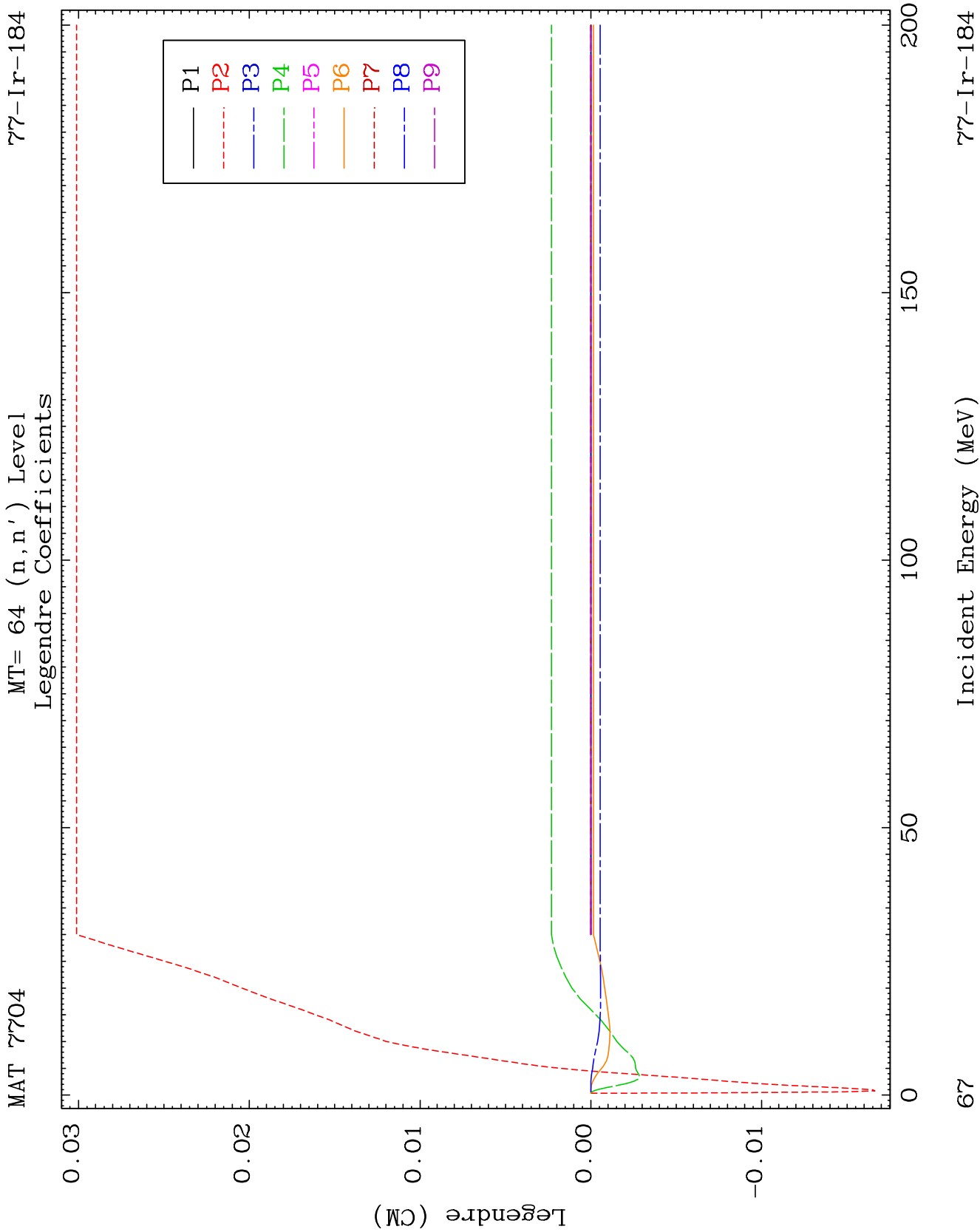
77-Ir-184

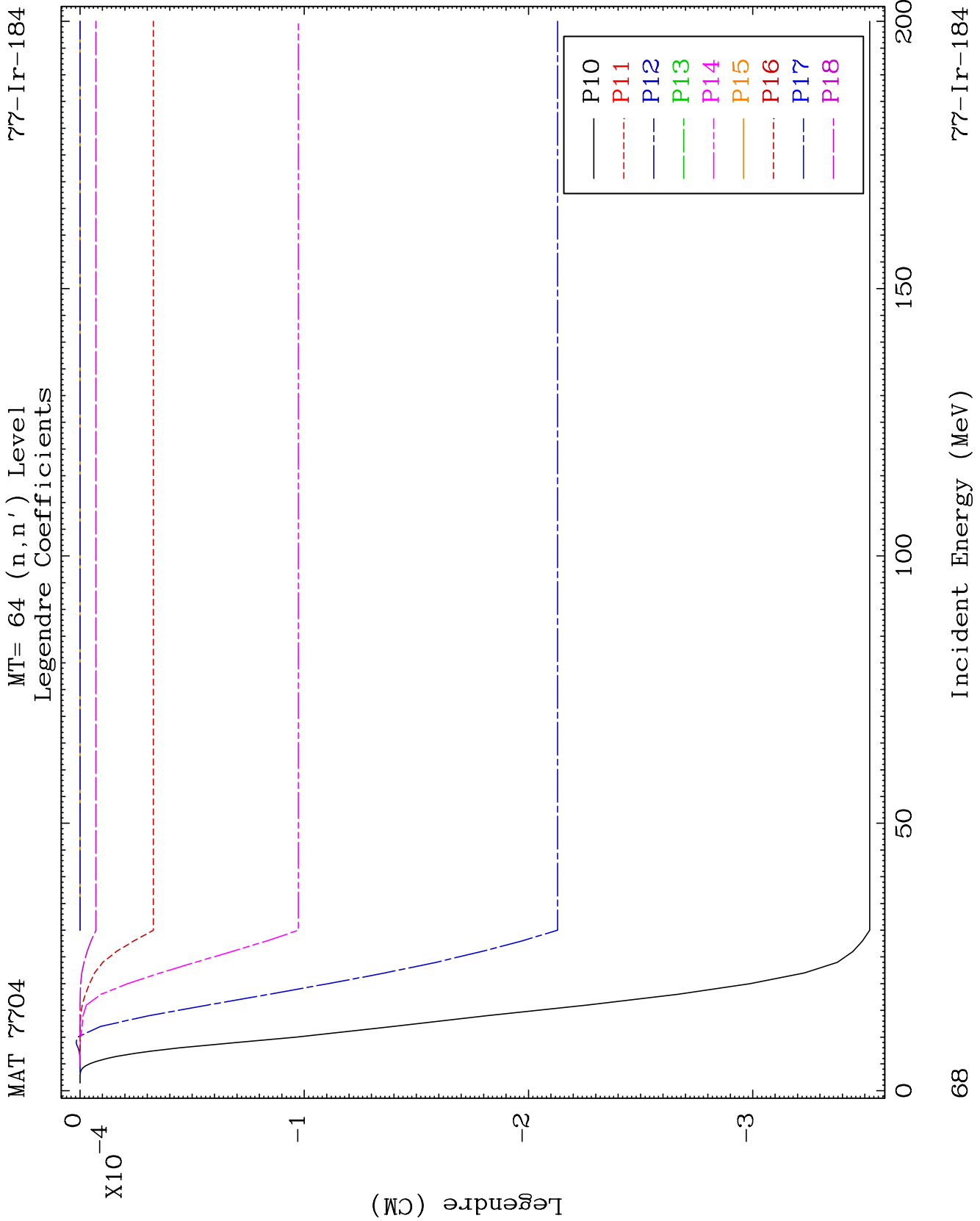


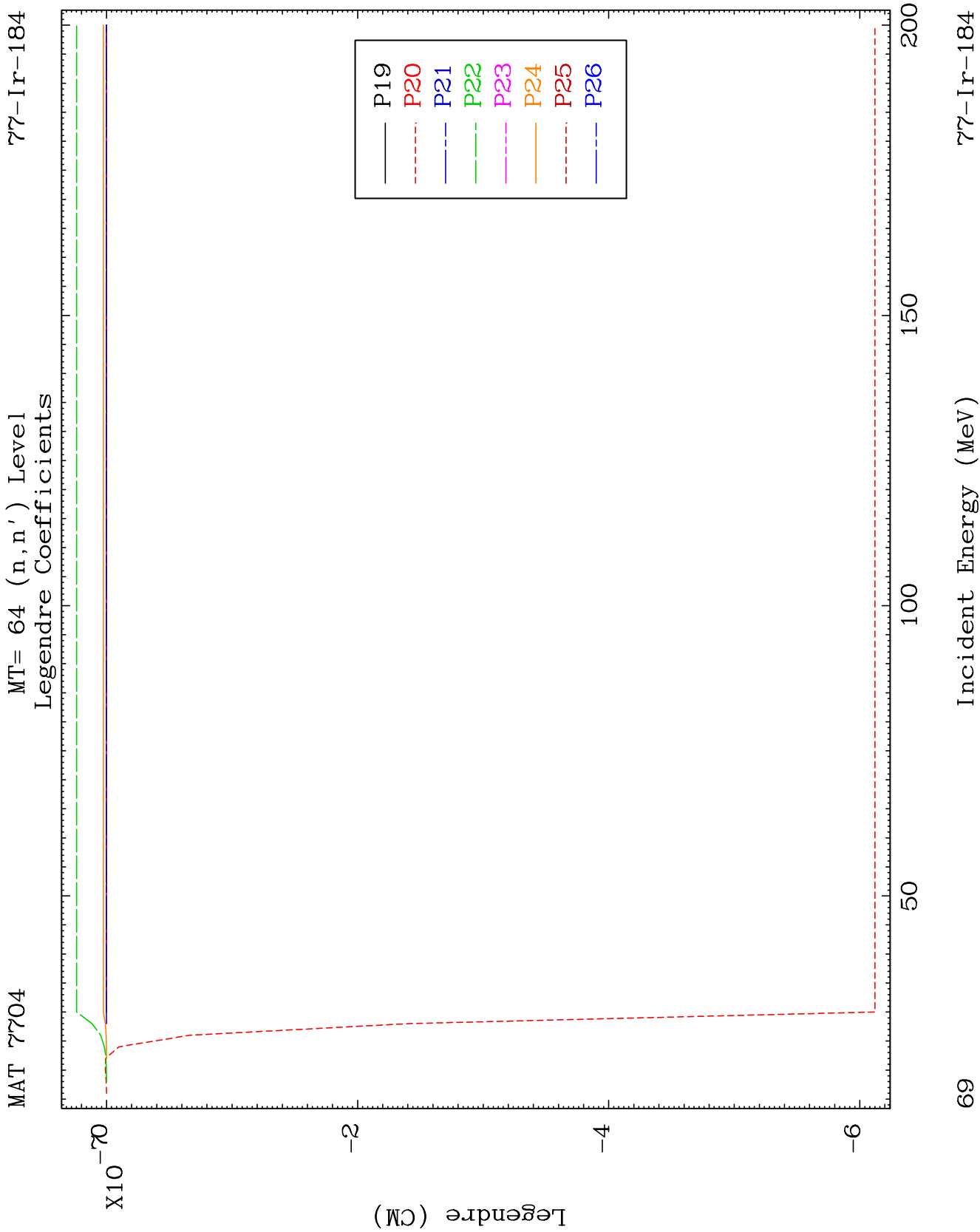


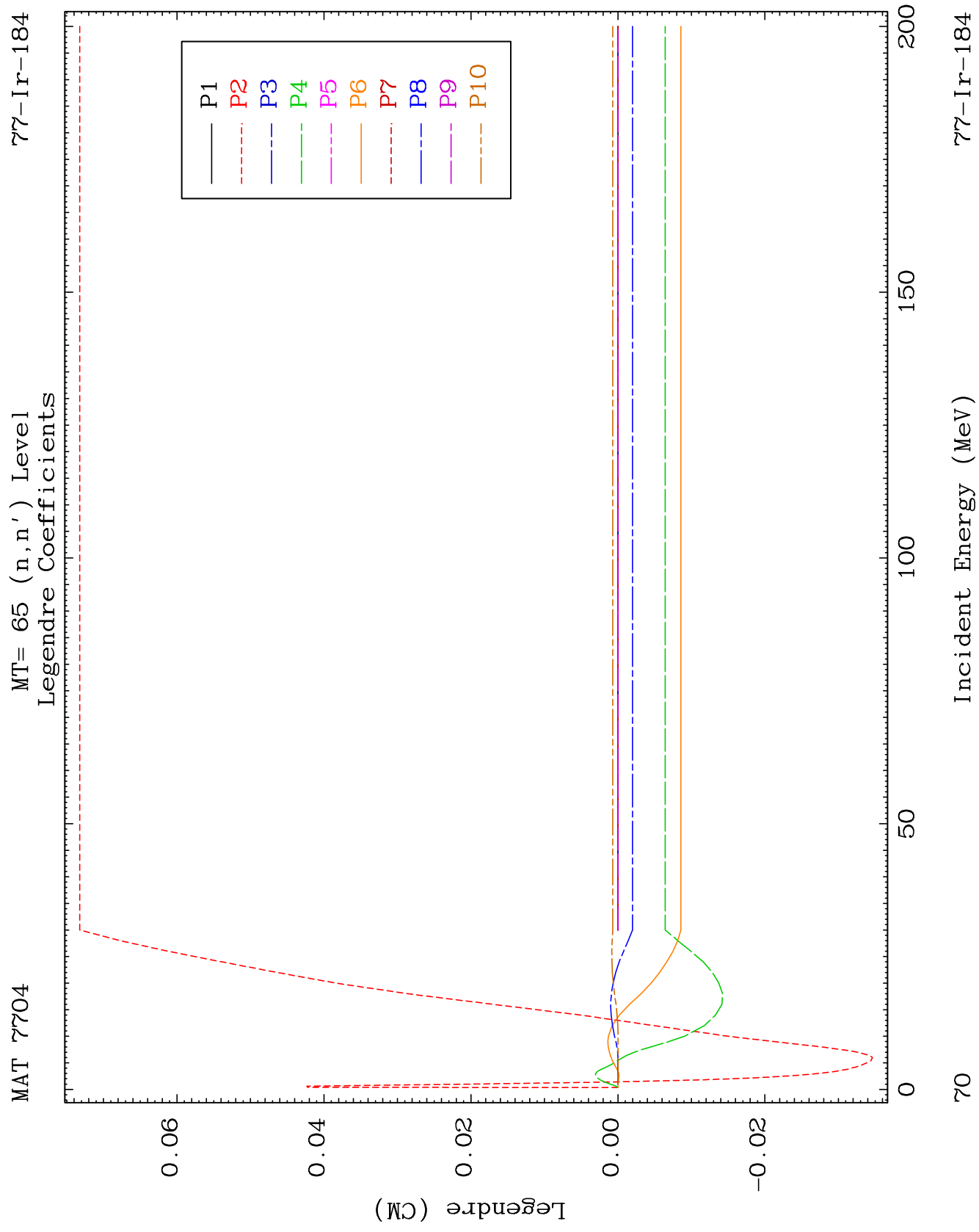


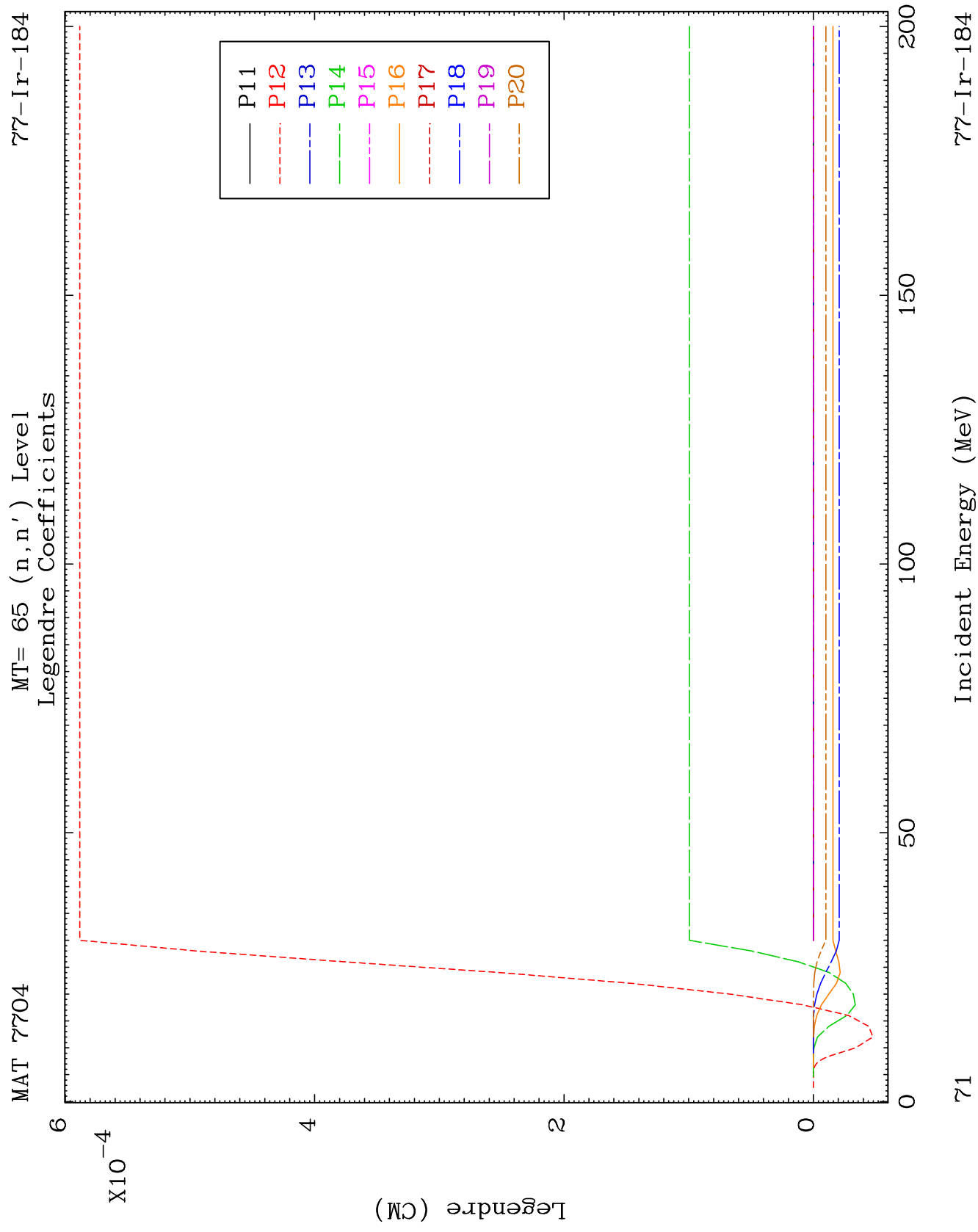


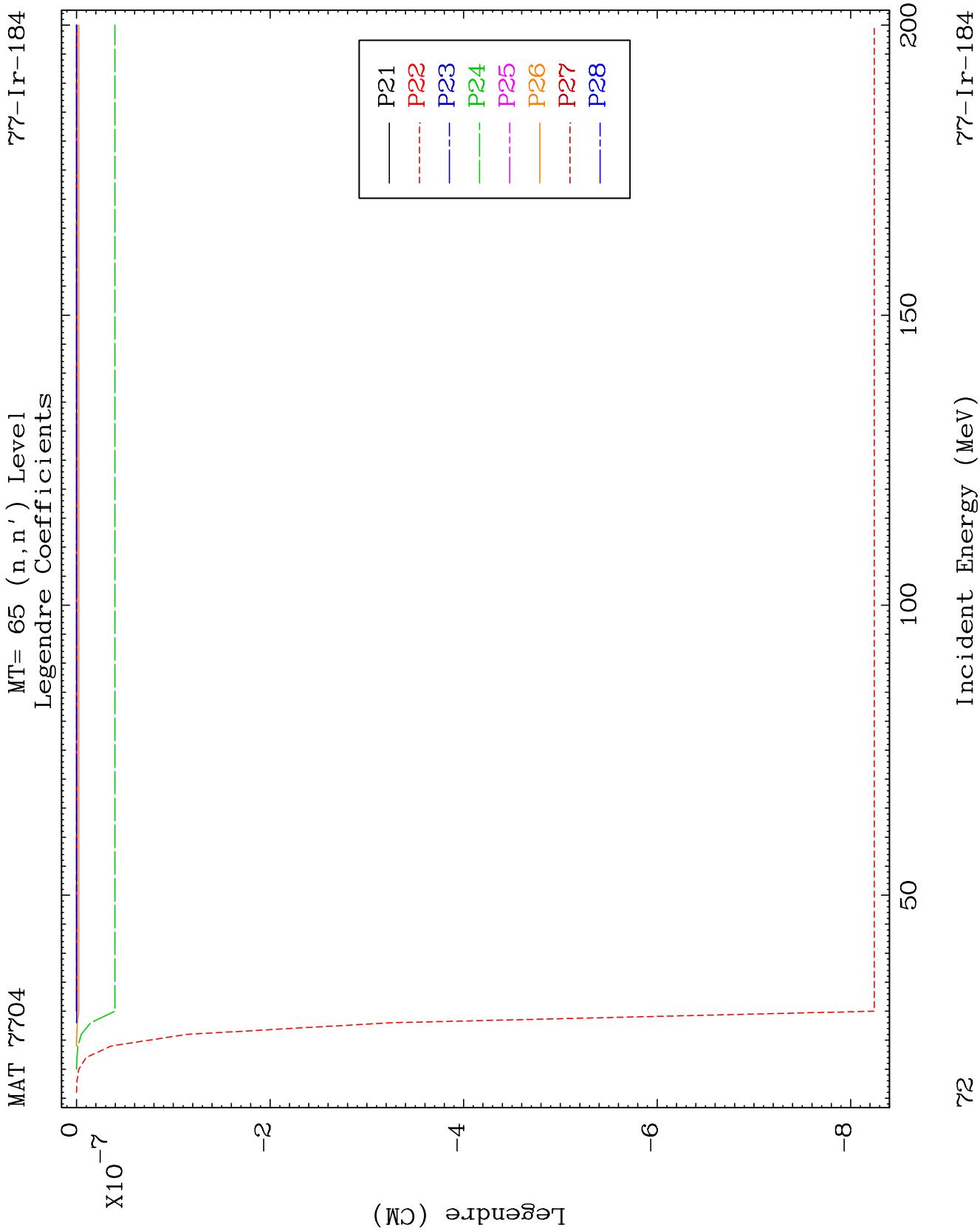


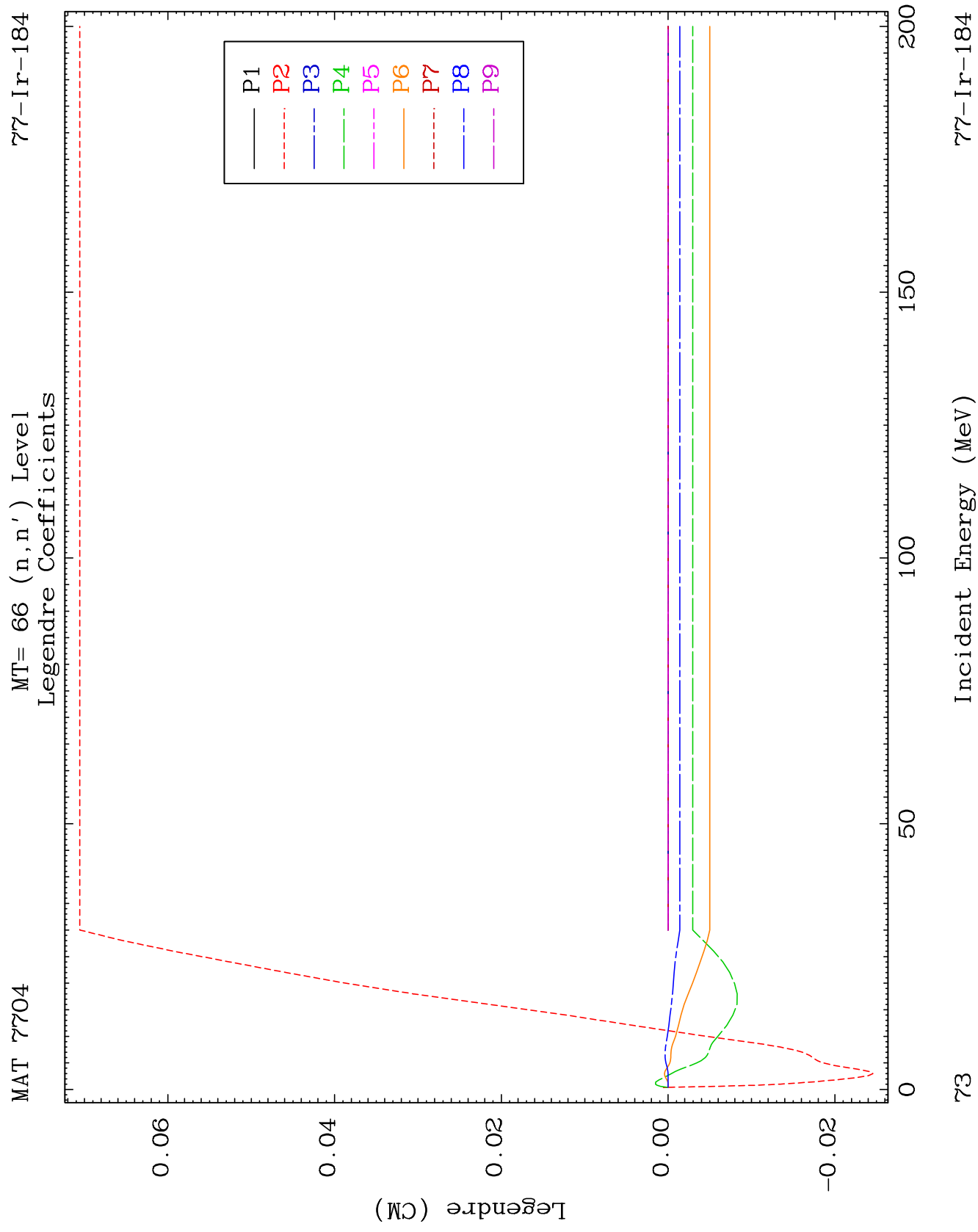


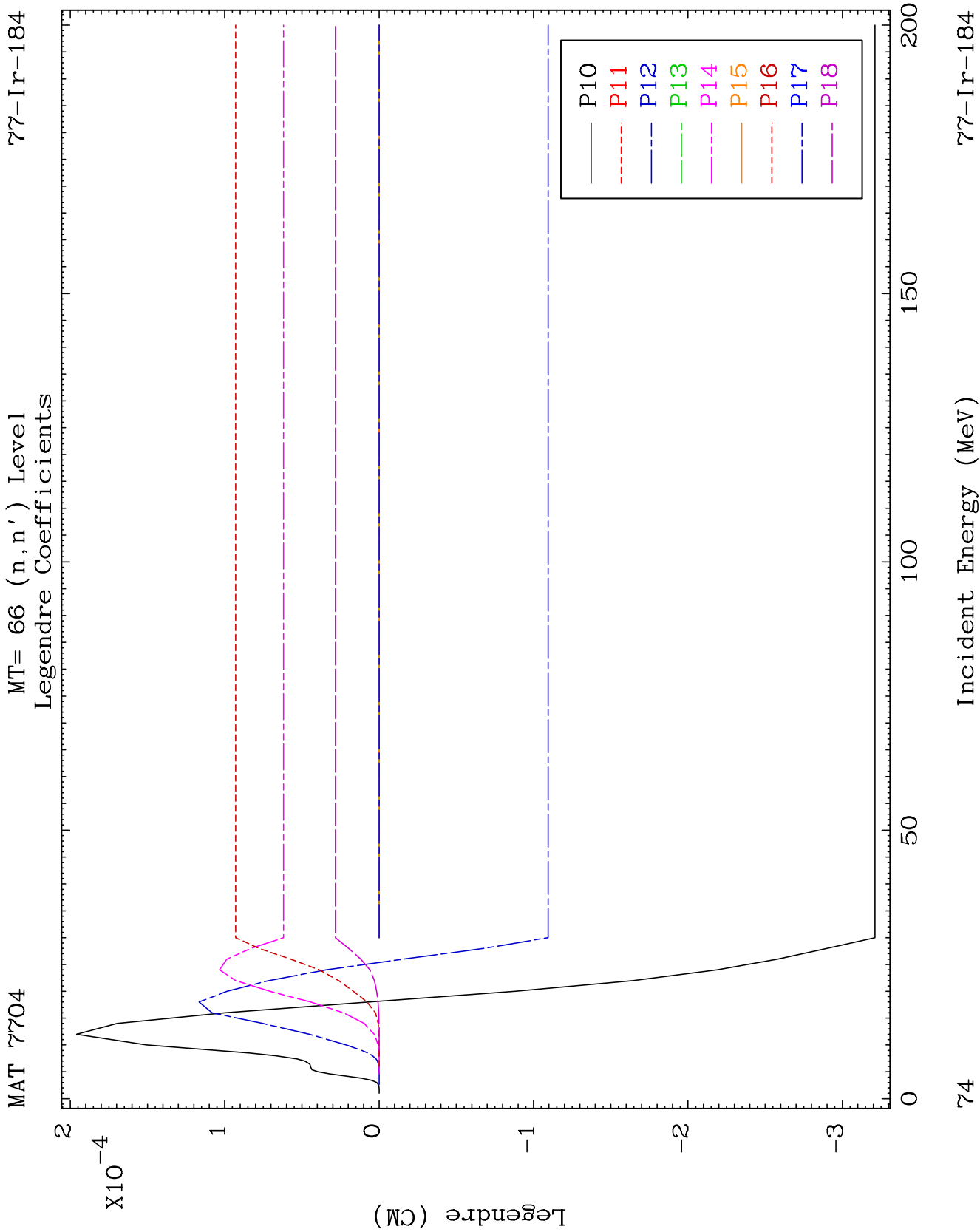


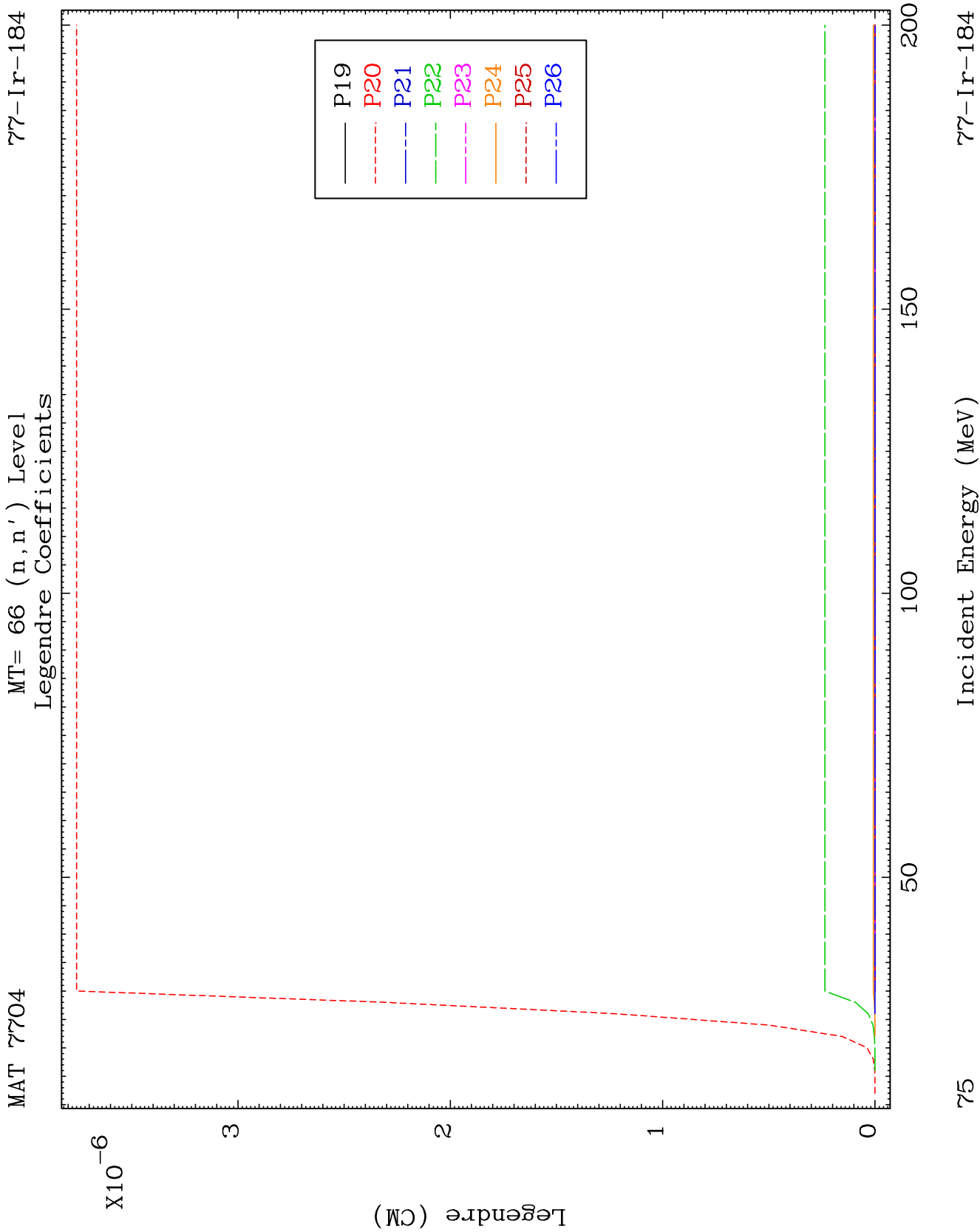


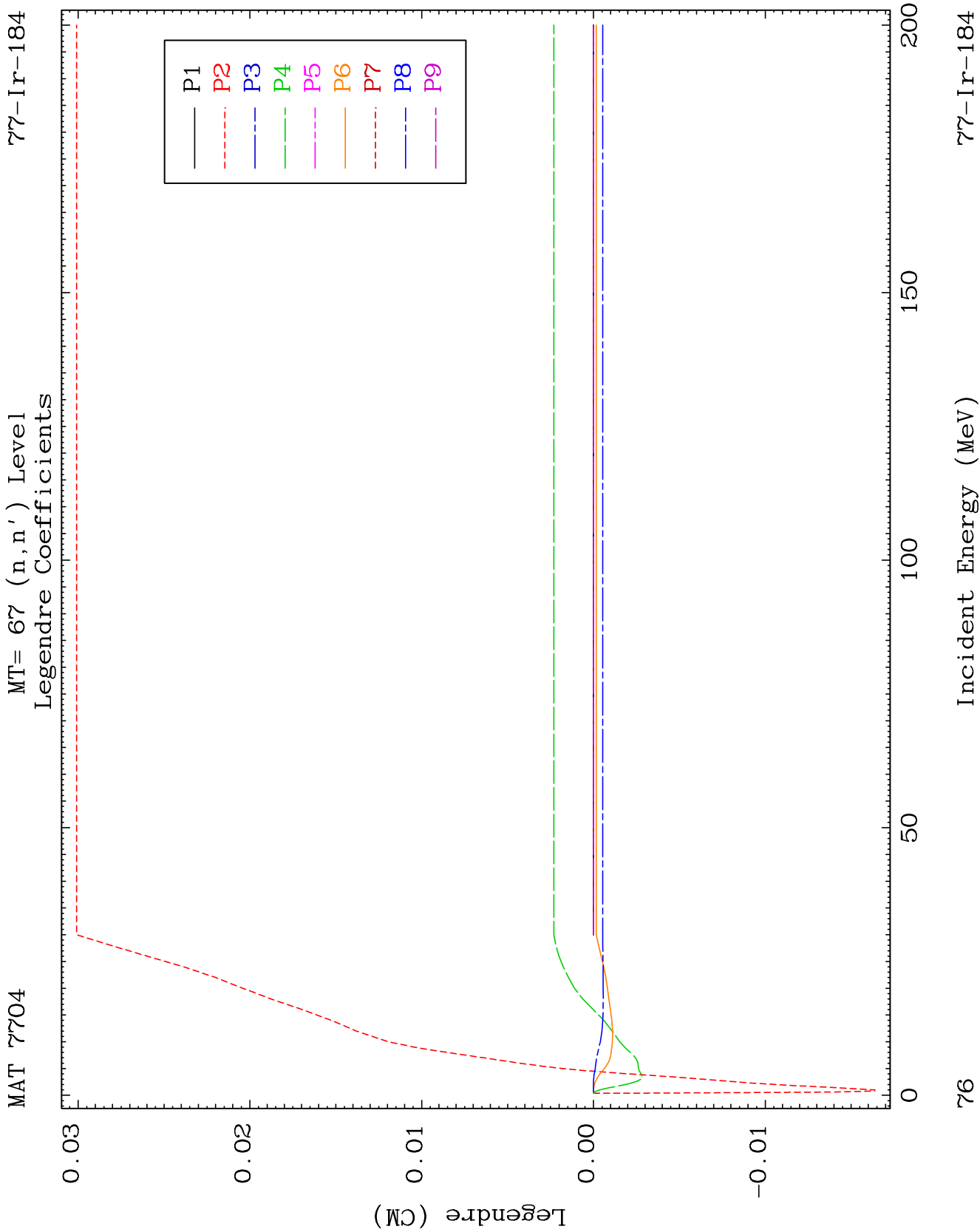


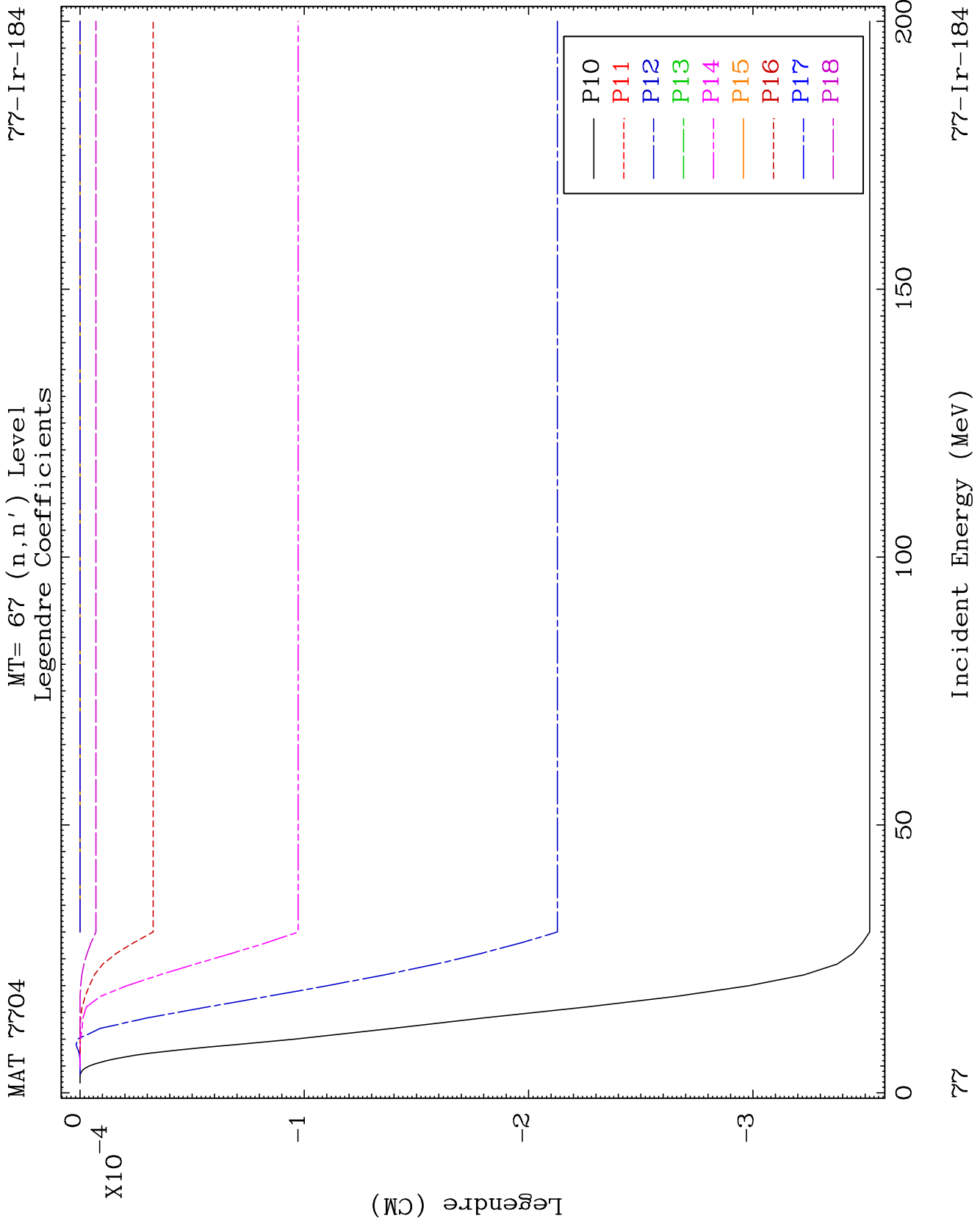


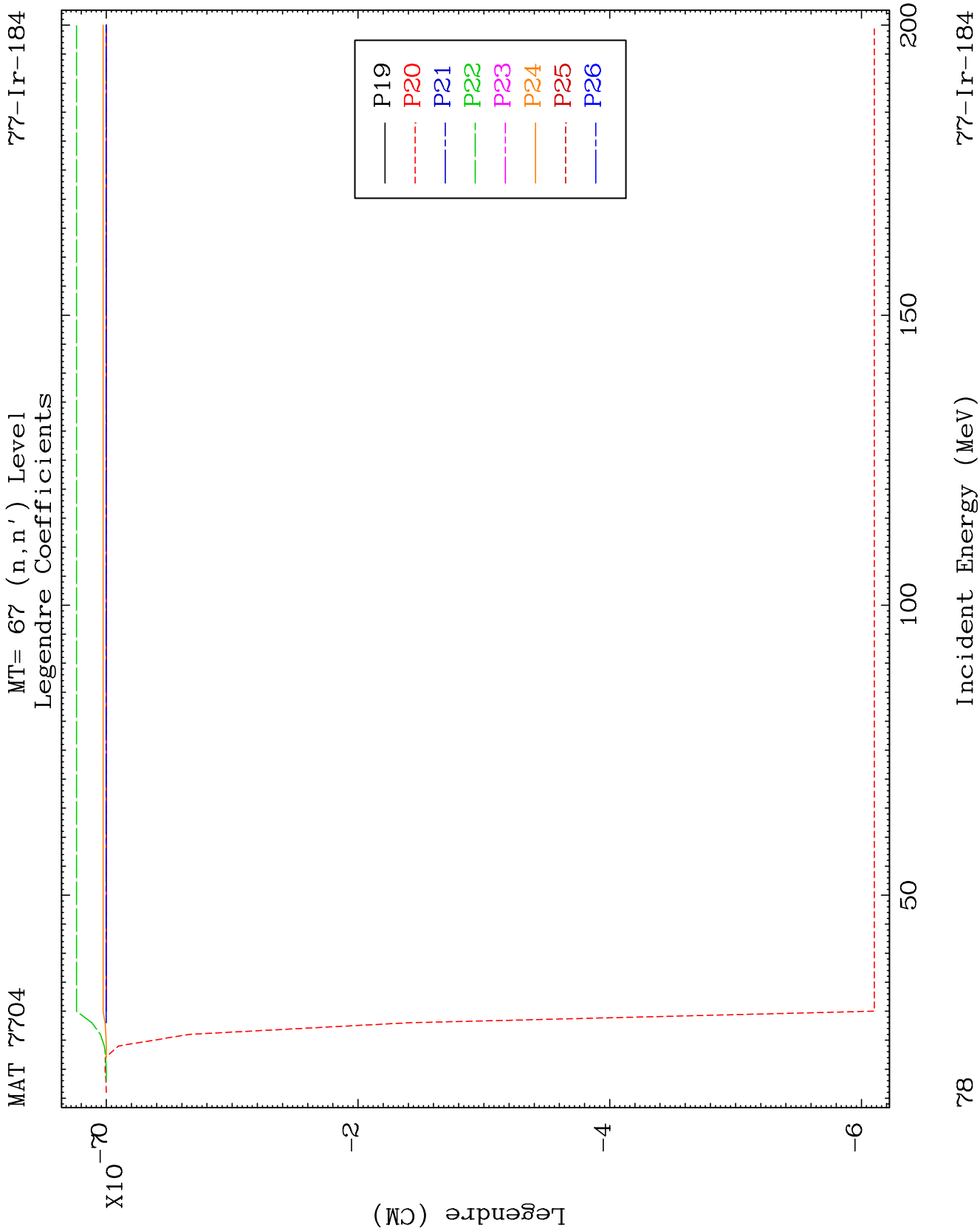








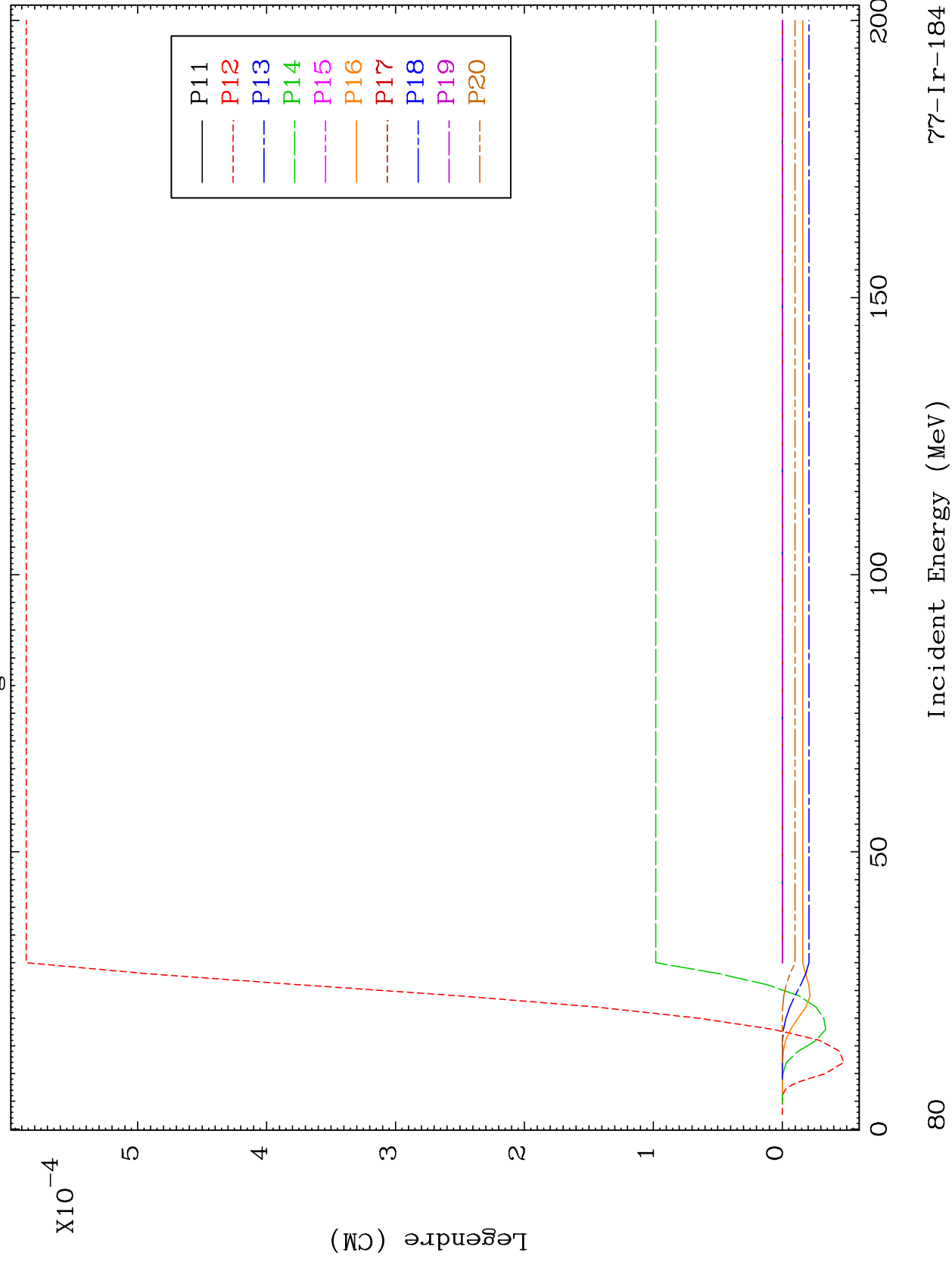


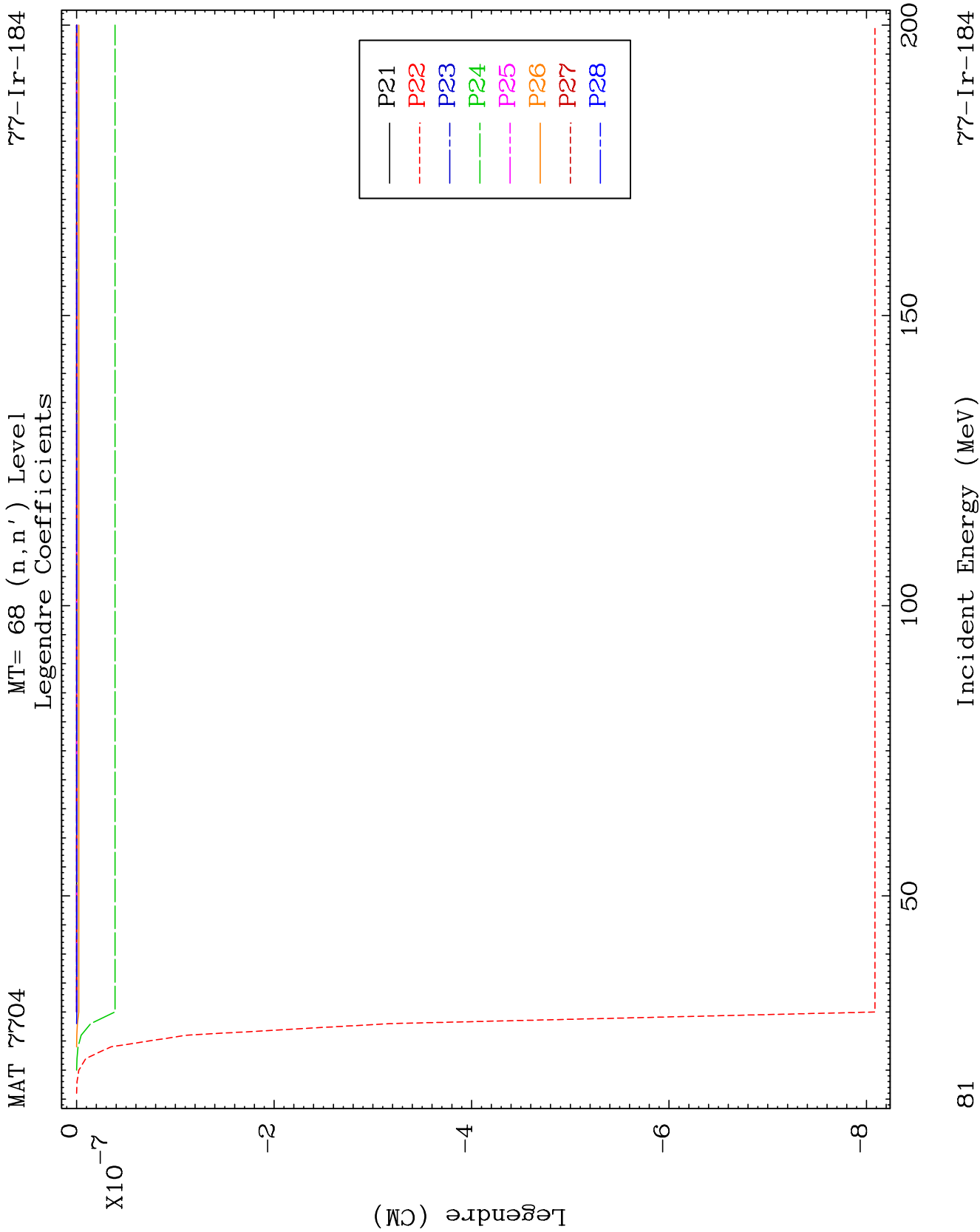


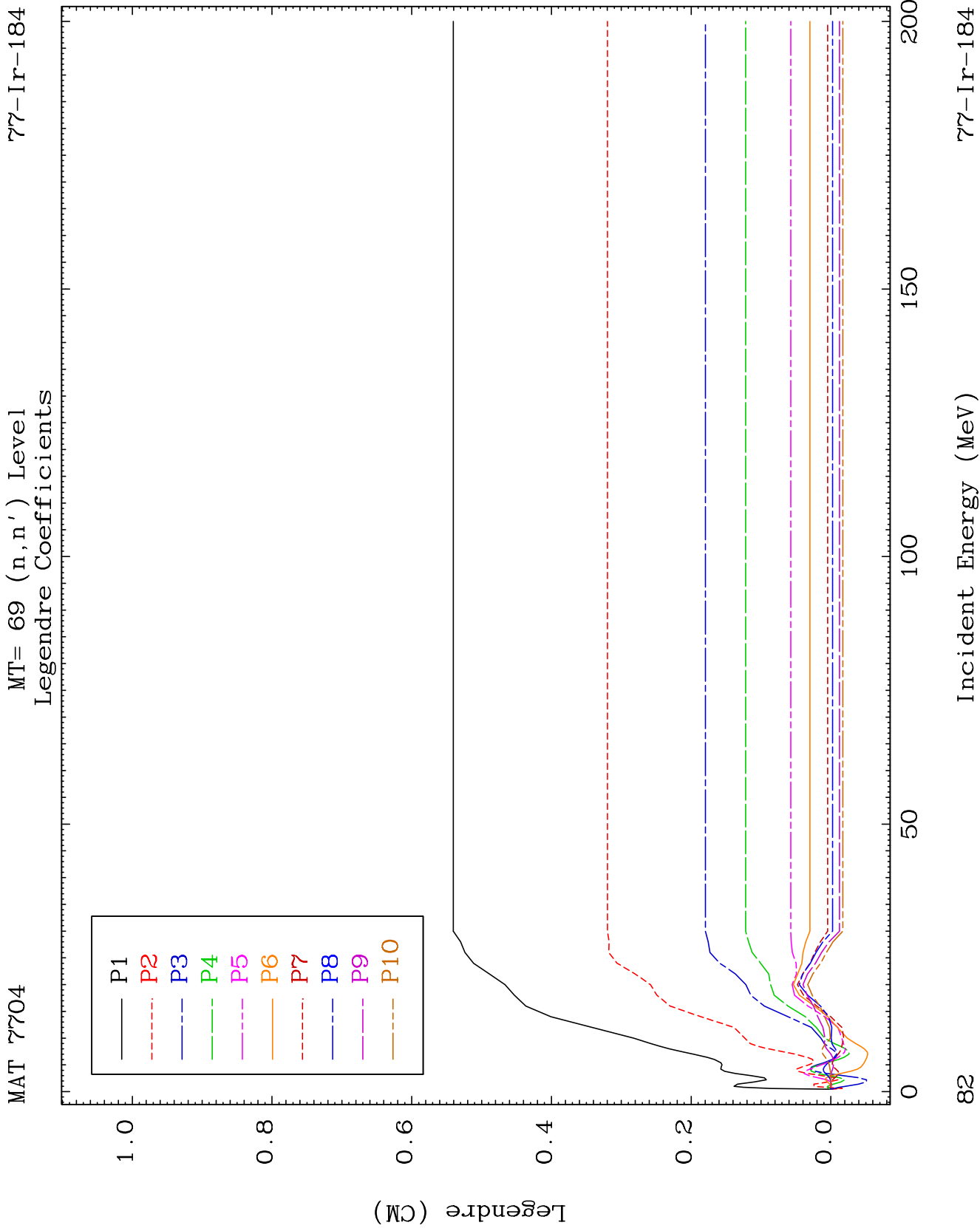
MAT 2704

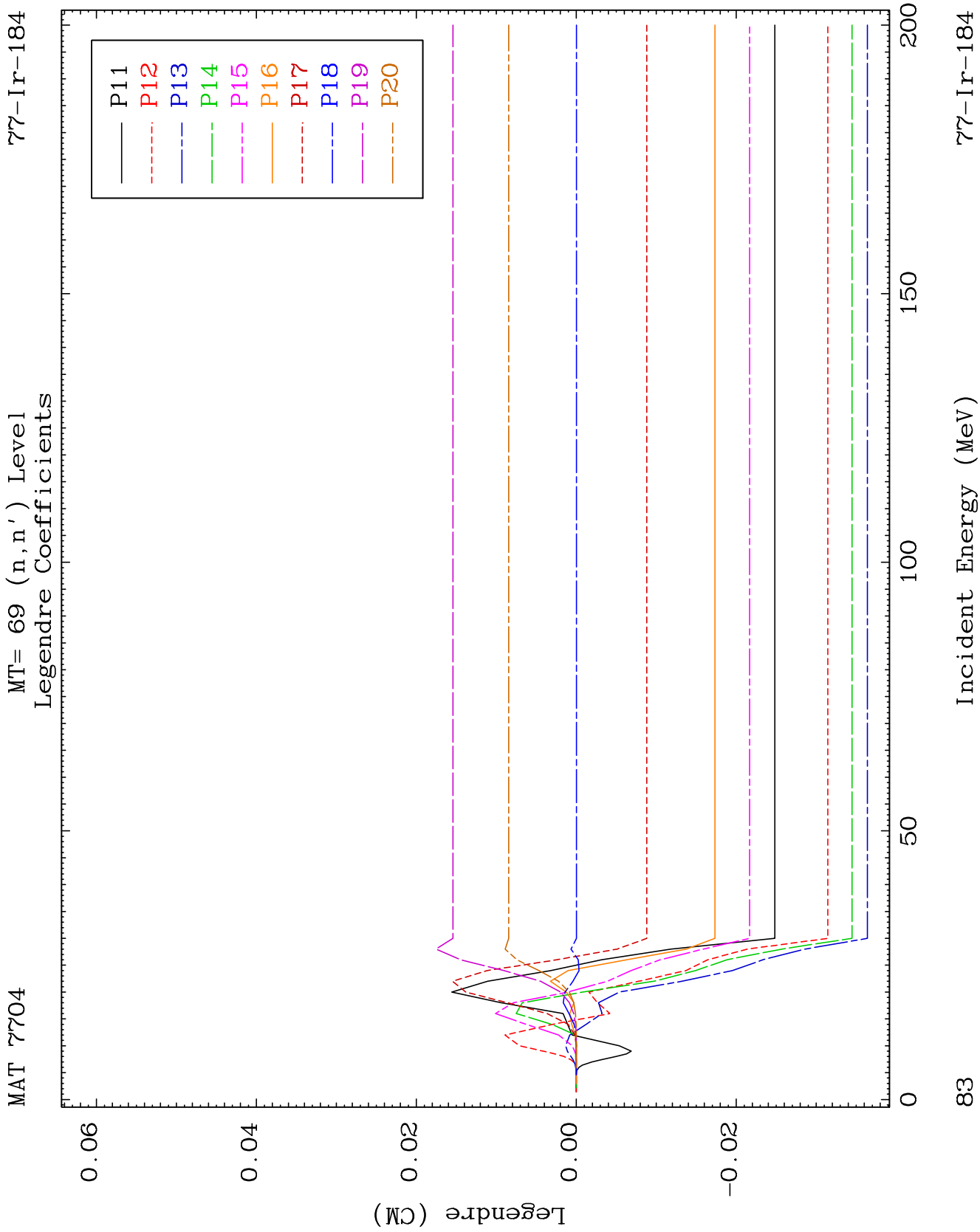
MT= 68 (n, n') Level
Legendre Coefficients

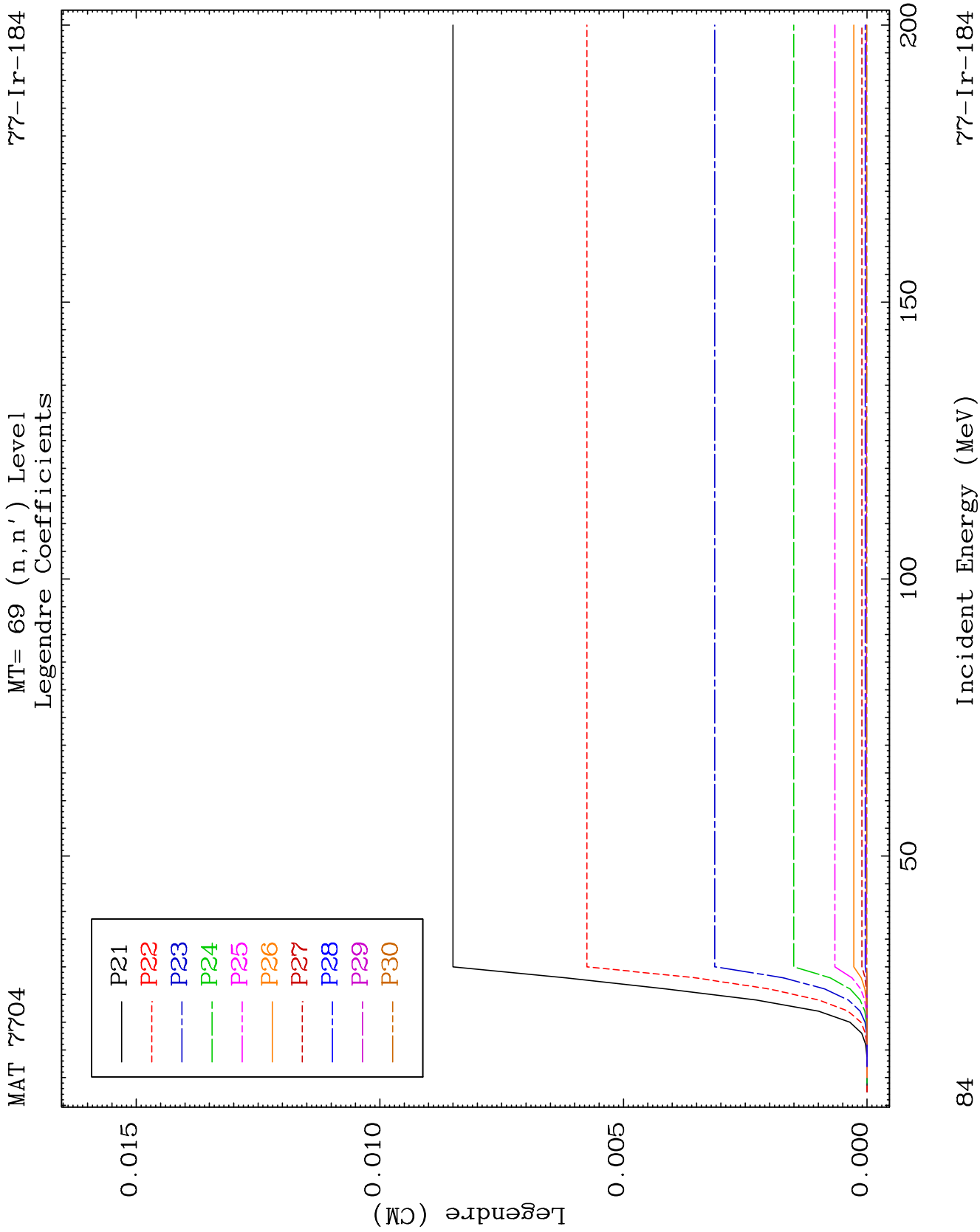
77-Ir-184

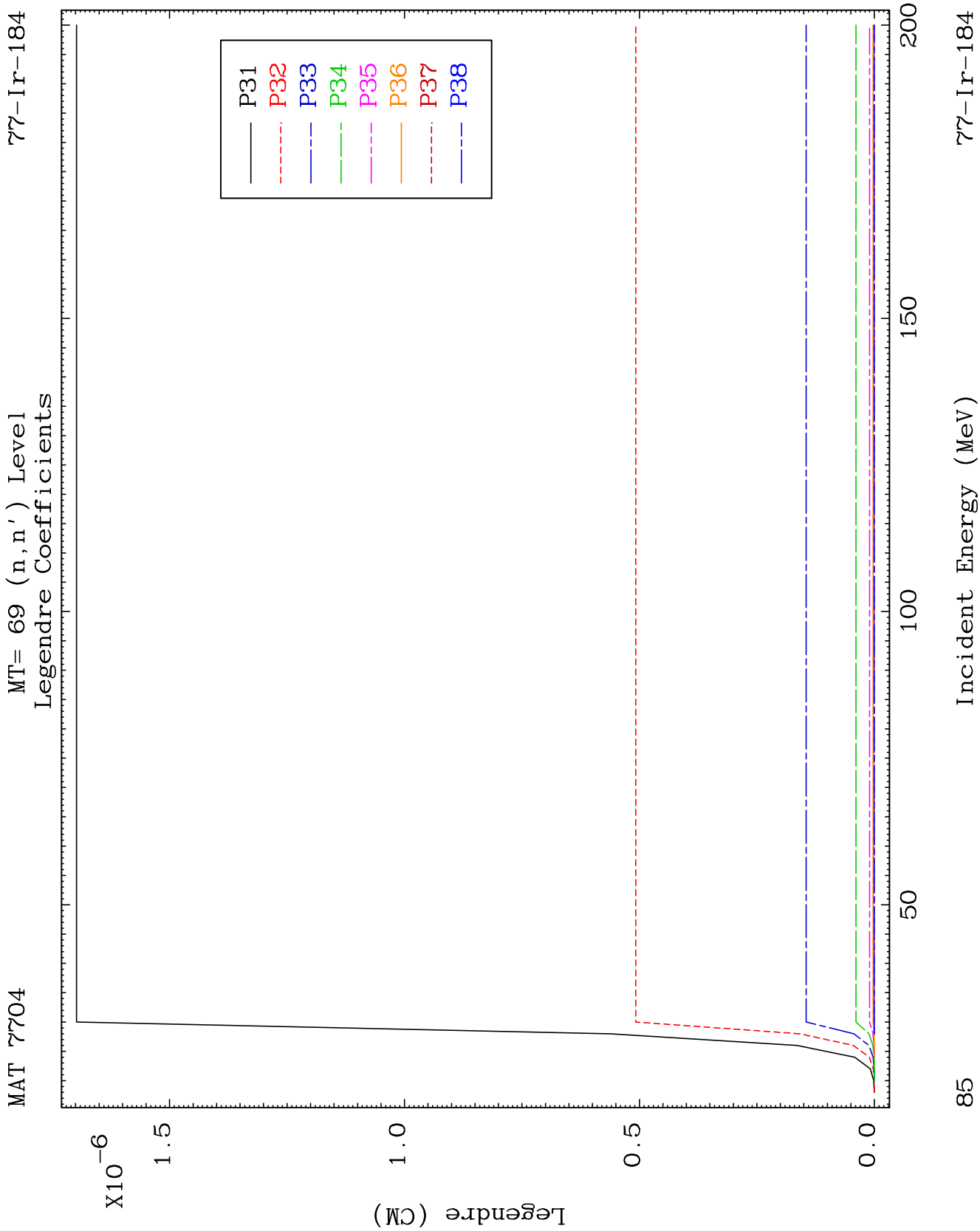


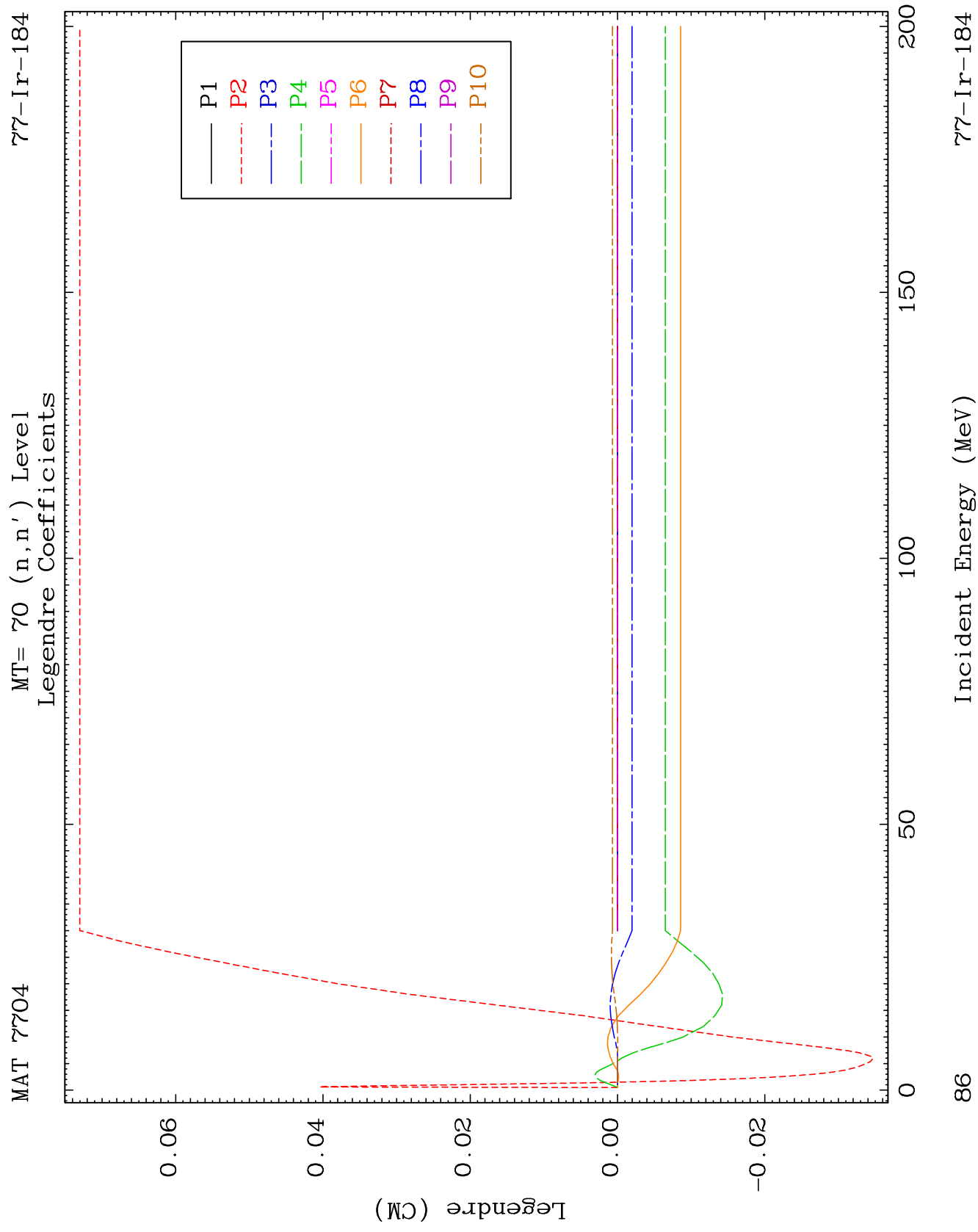


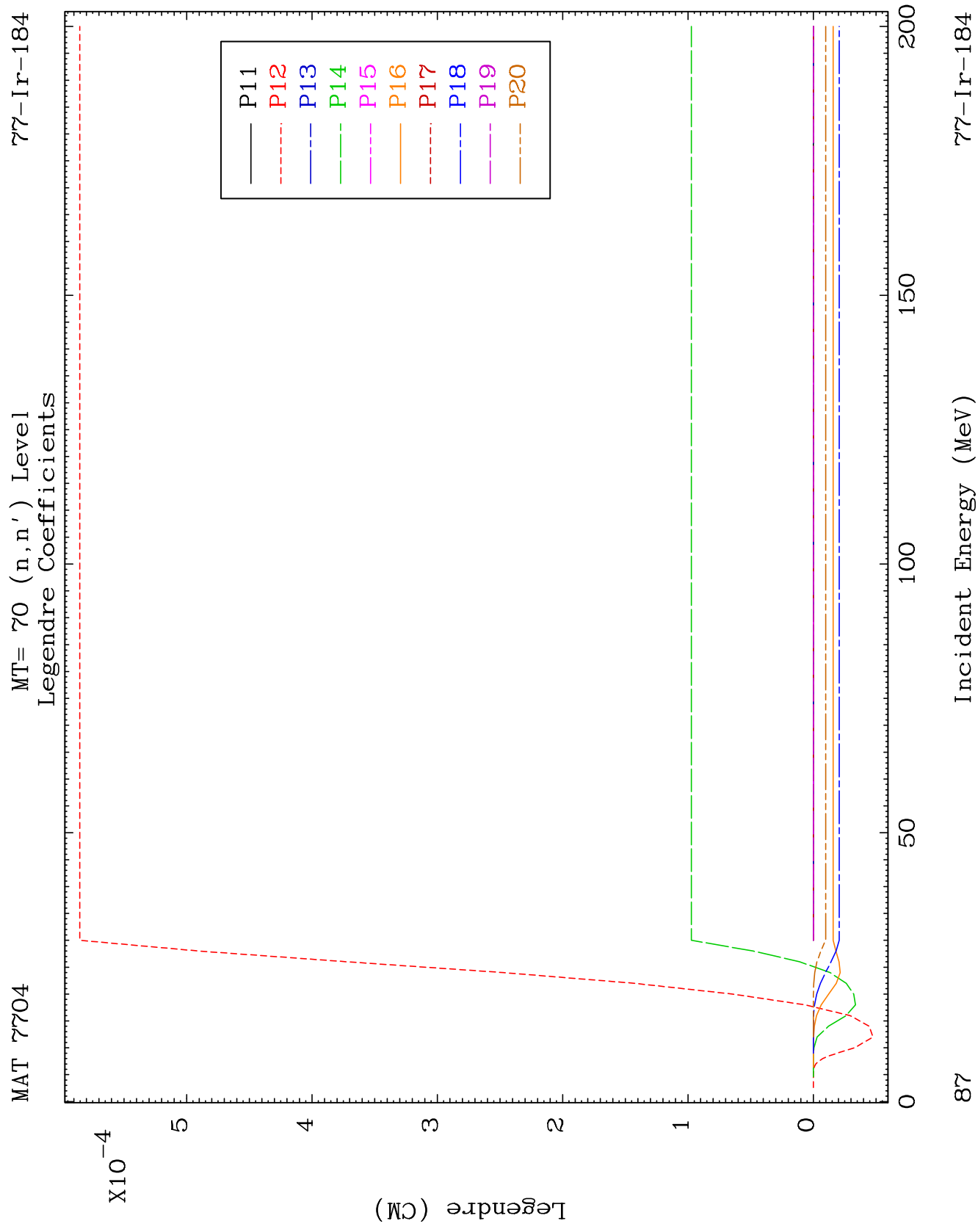


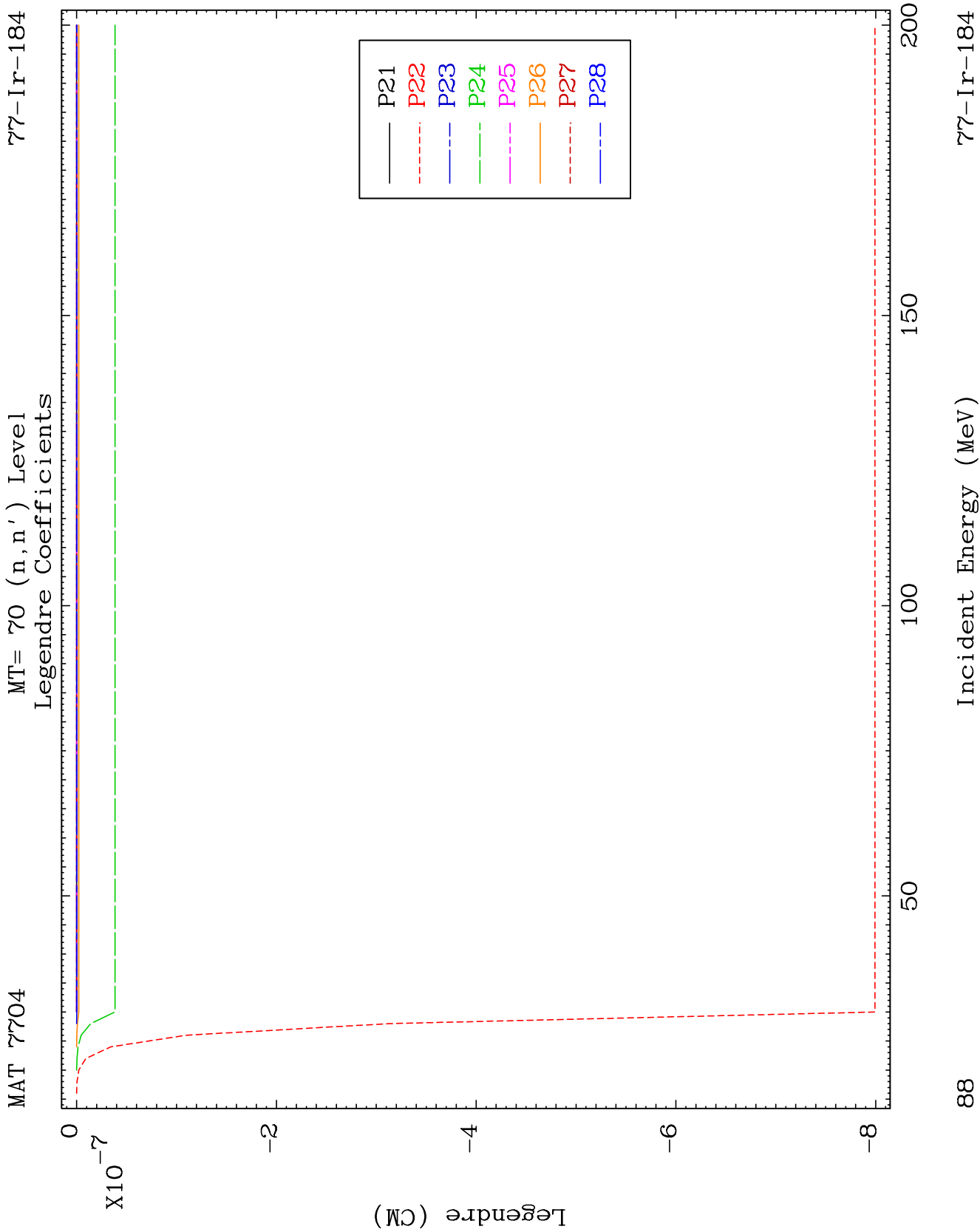








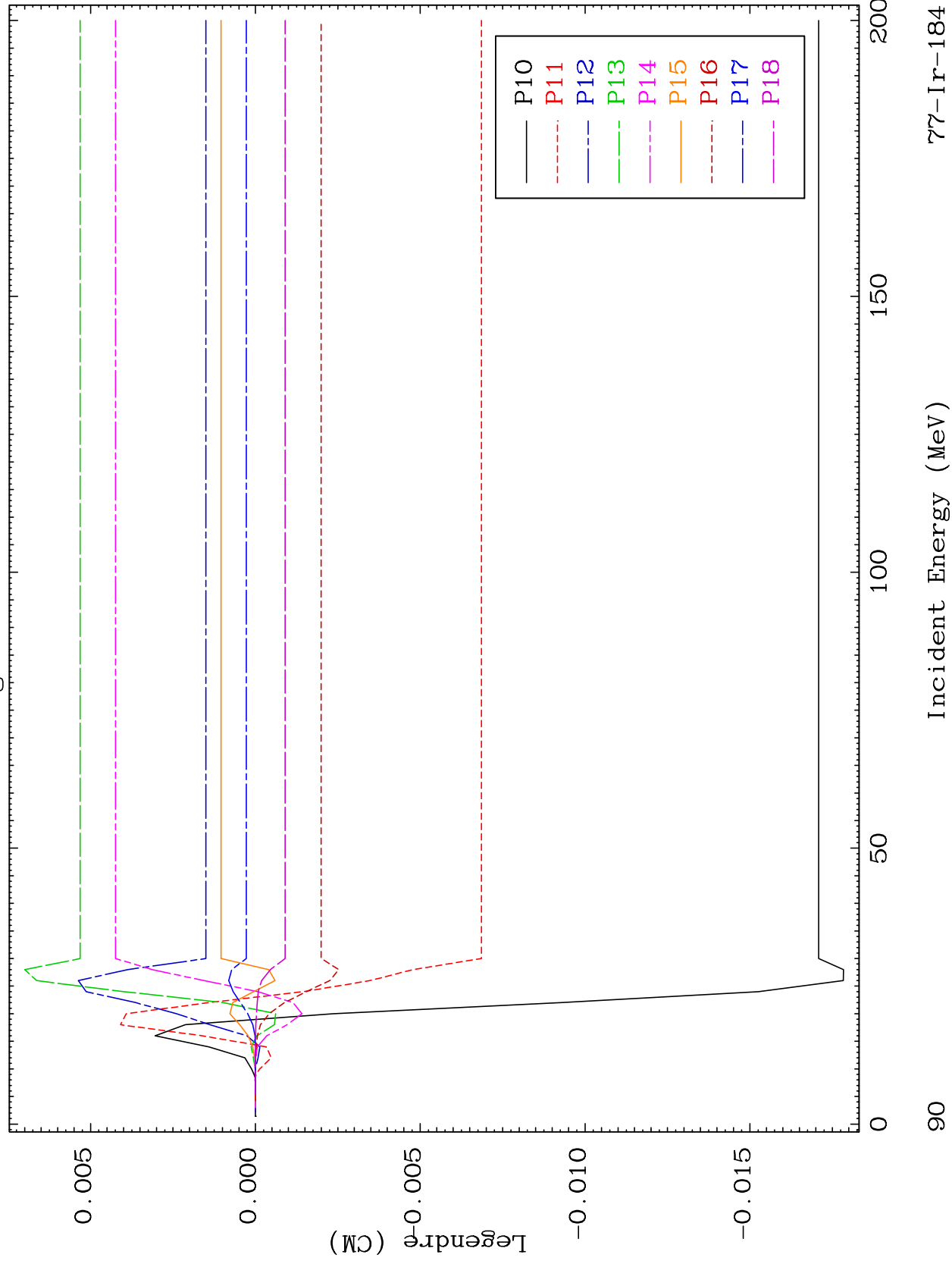


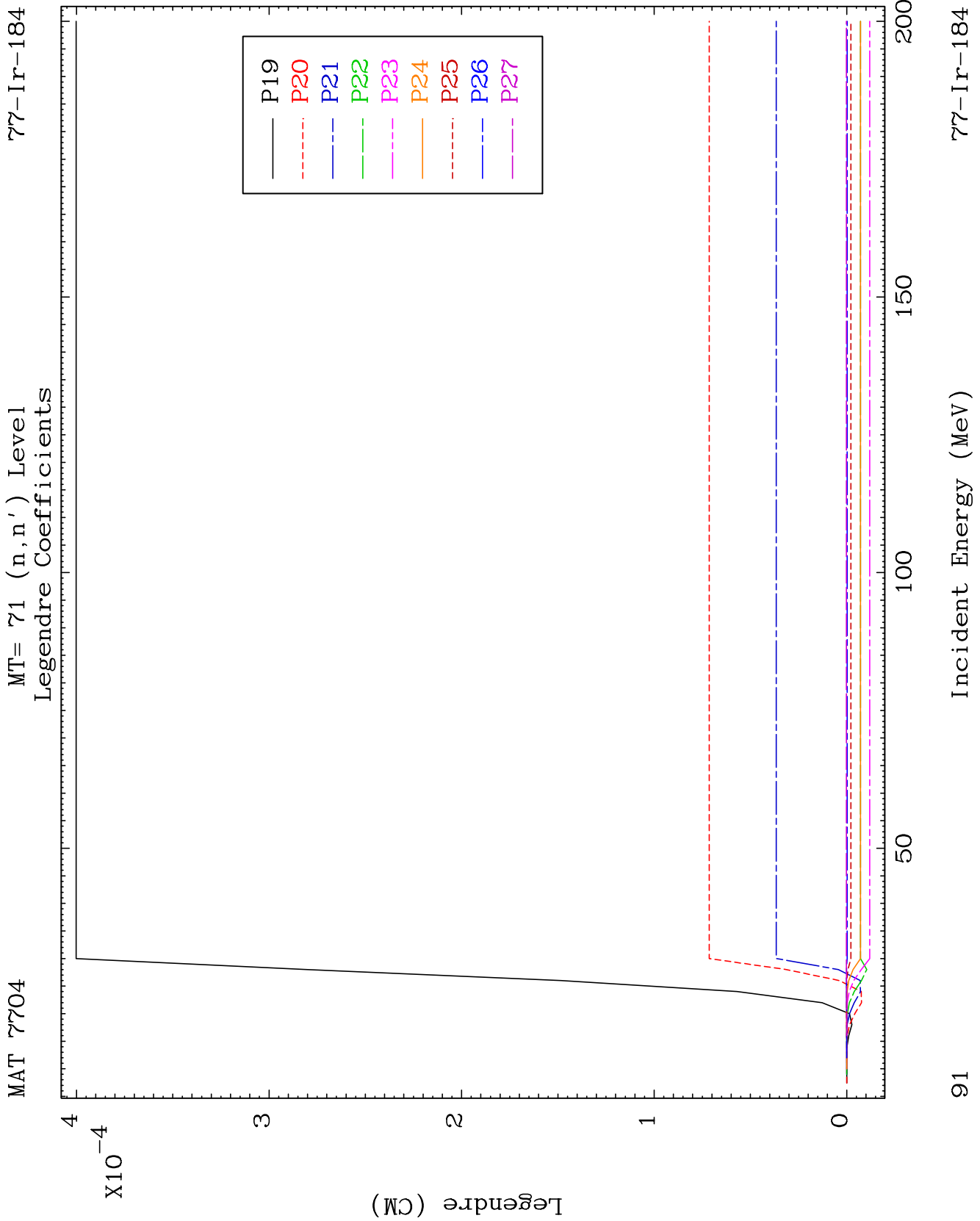


MAT 2704

MT= 71 (n, n') Level
Legendre Coefficients

77-Ir-184

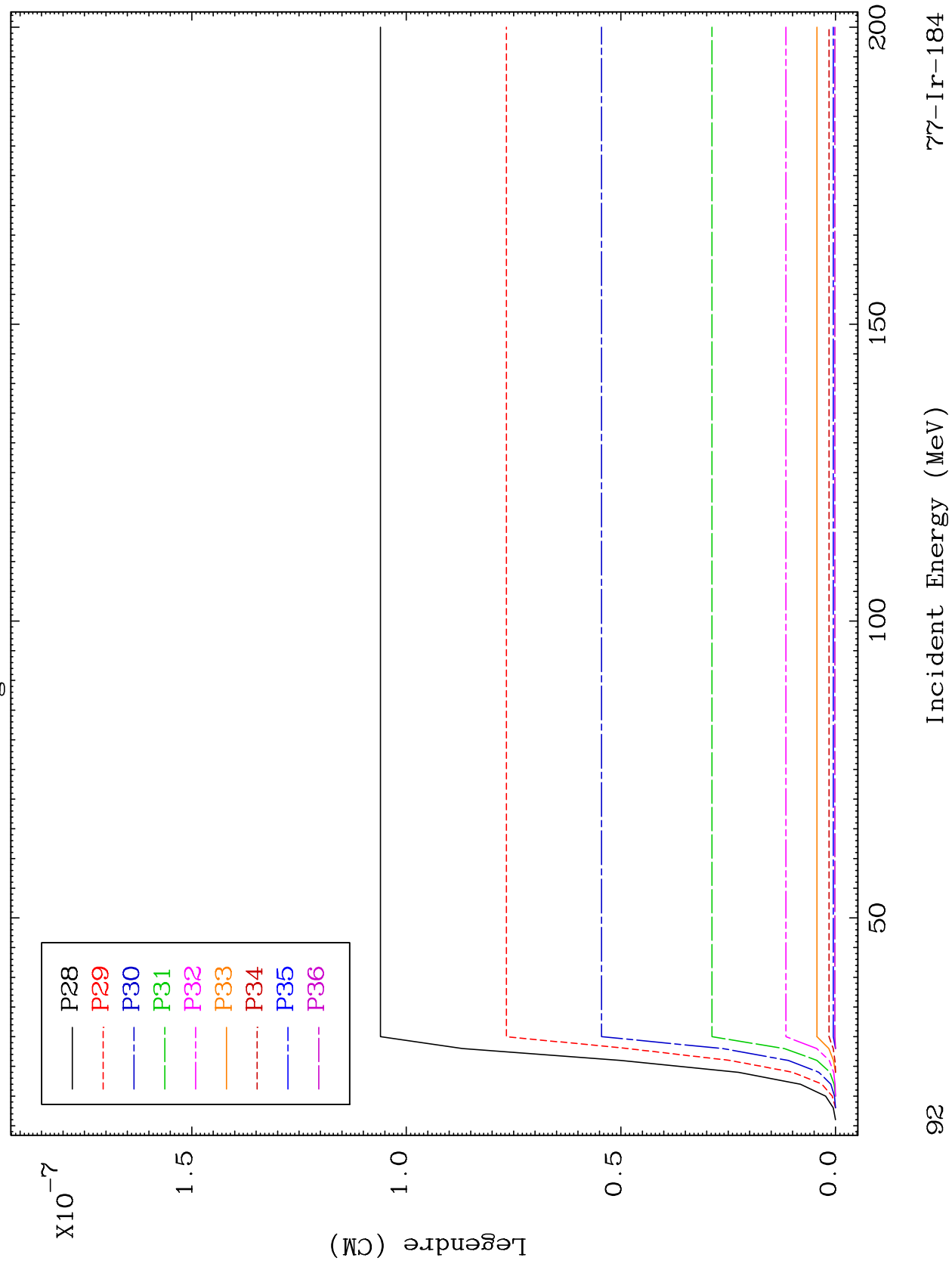




MAT 2704

MT= 71 (n, n') Level
Legendre Coefficients

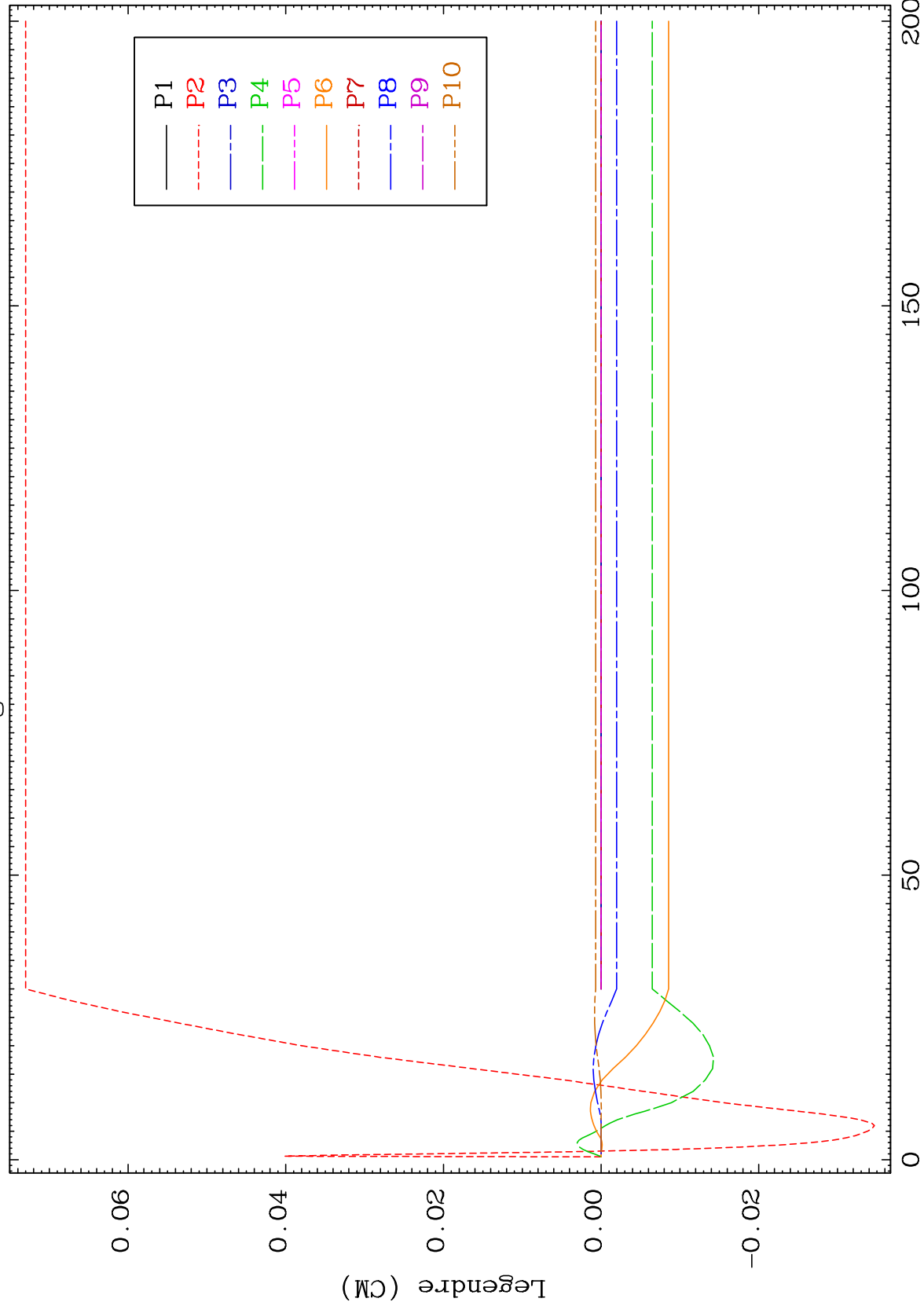
77-Ir-184



MAT 2704

MT= 72 (n, n') Level
Legendre Coefficients

77-Ir-184



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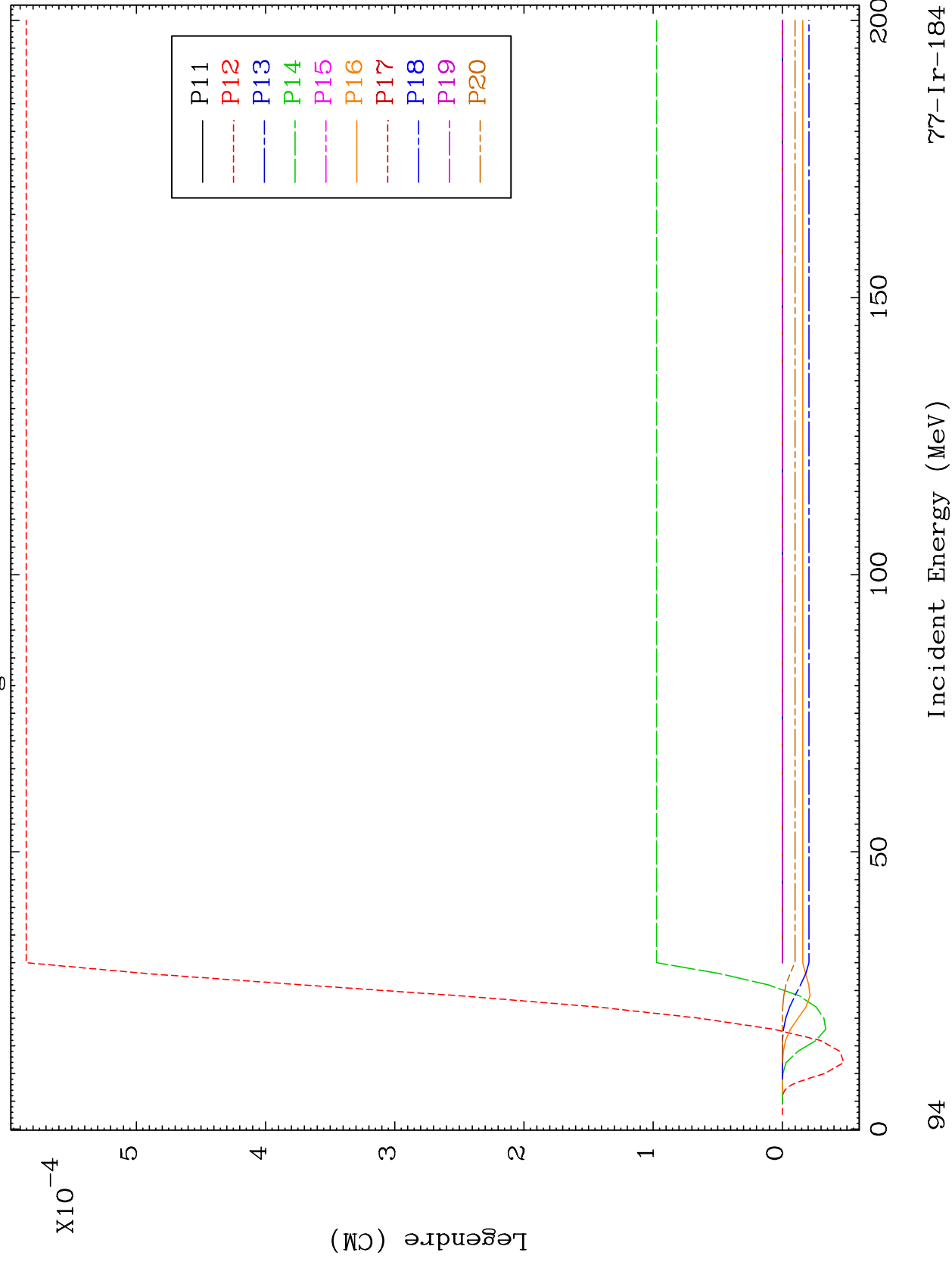
Incident Energy (MeV)

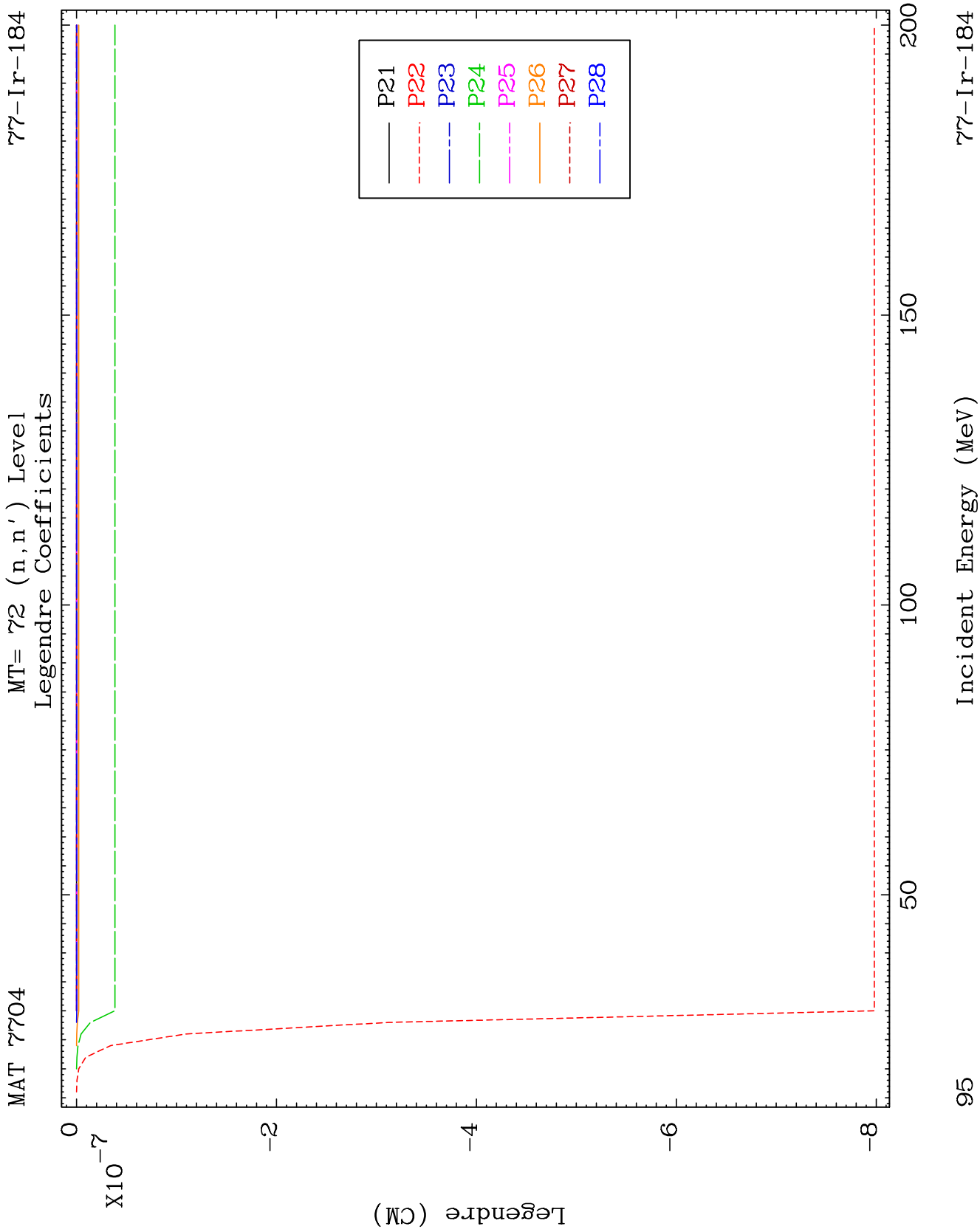
77-1r-184

MAT 7704

MT= 72 (n, n') Level
Legendre Coefficients

77-Ir-184

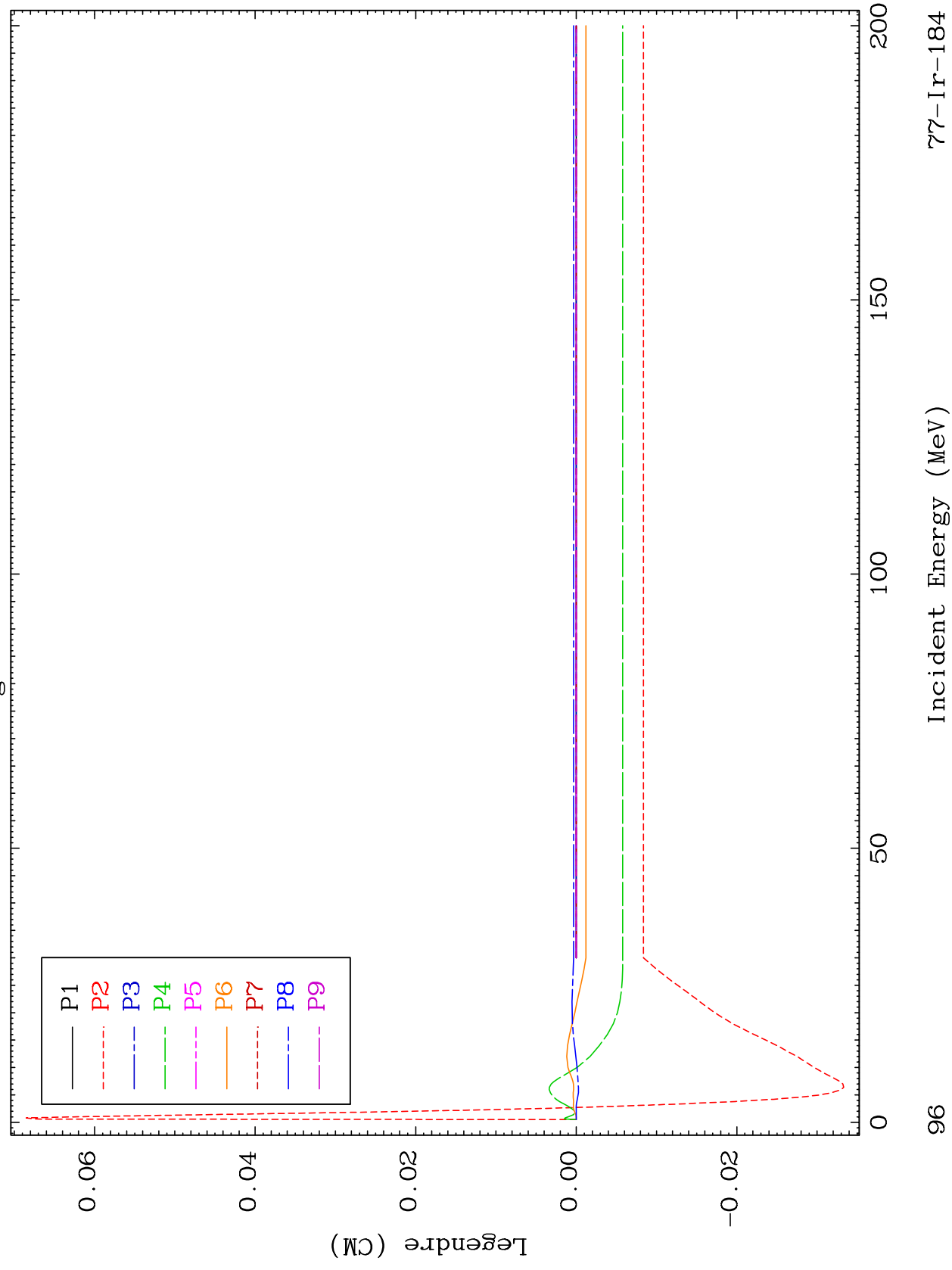


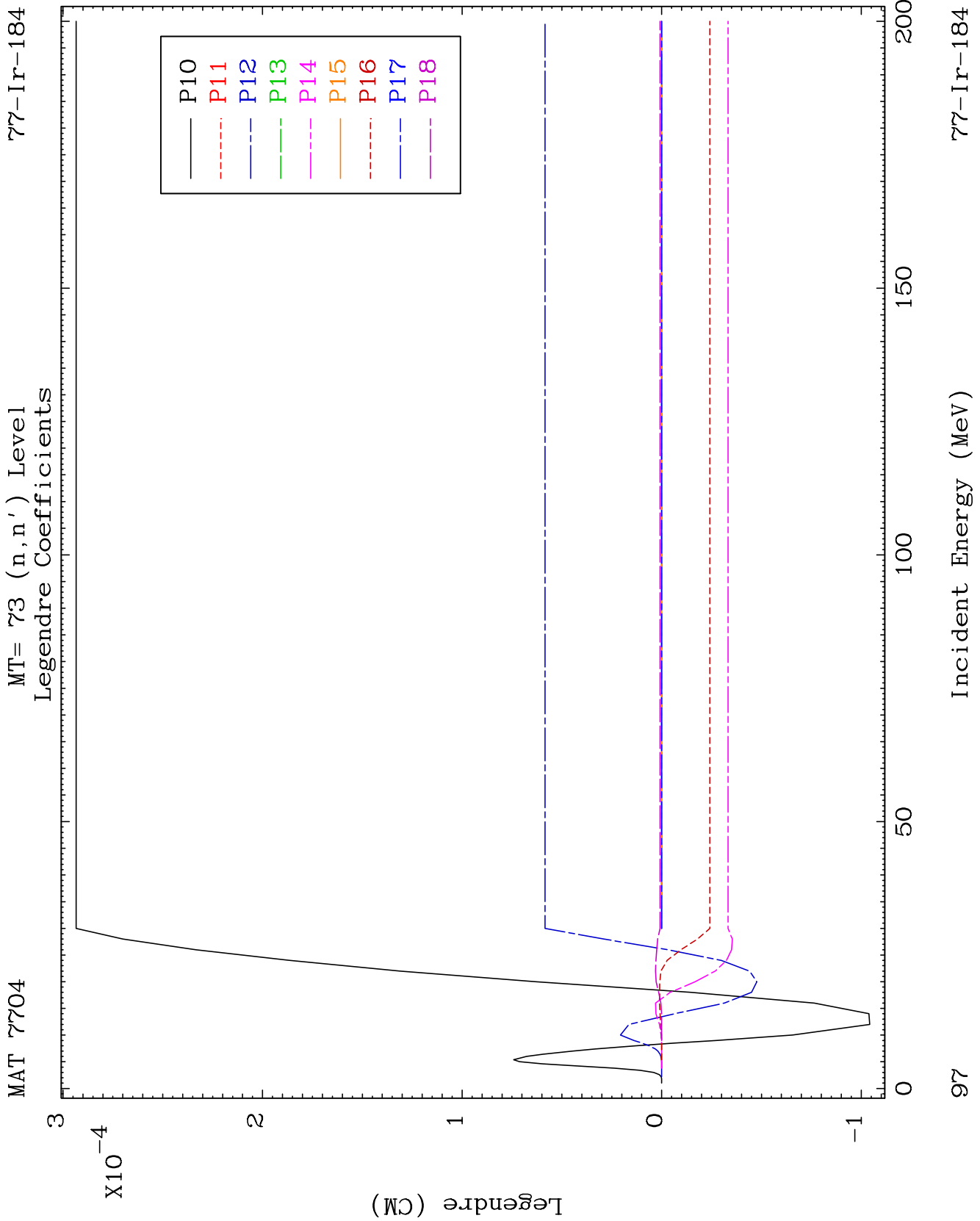


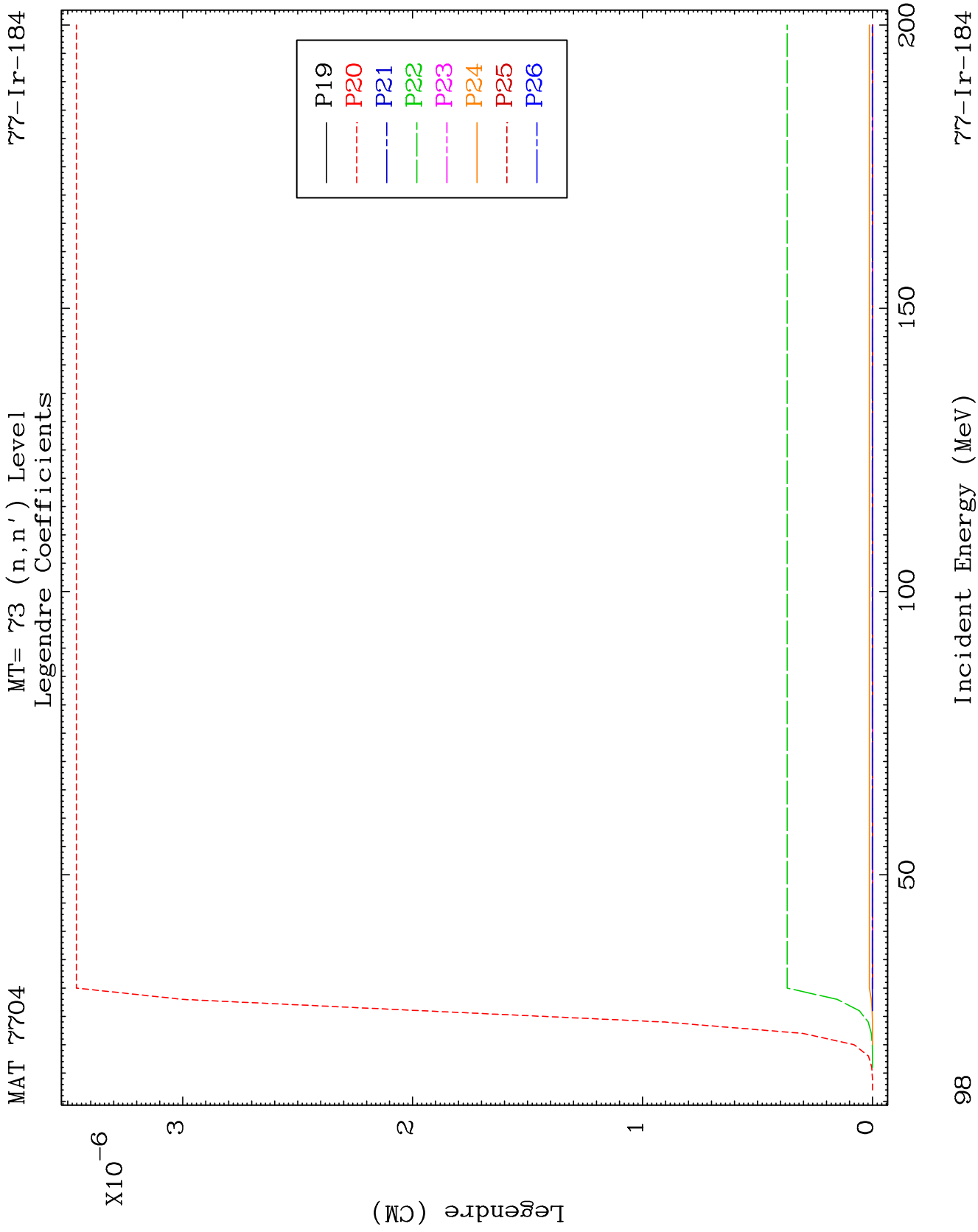
MAT 7704

MT= 73 (n, n') Level
Legendre Coefficients

77-Ir-184



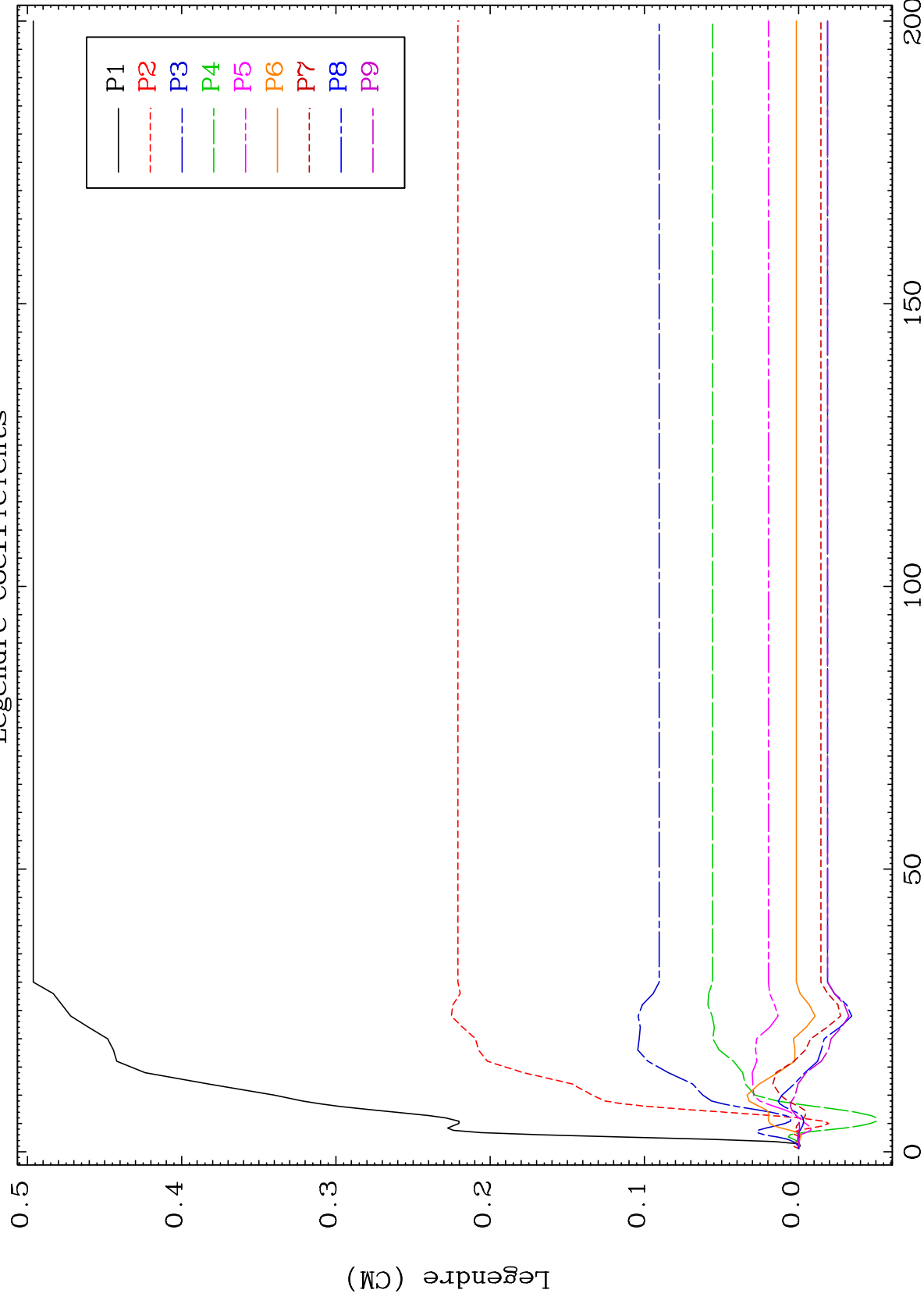




MAT 2704

MT= 74 (n, n') Level
Legendre Coefficients

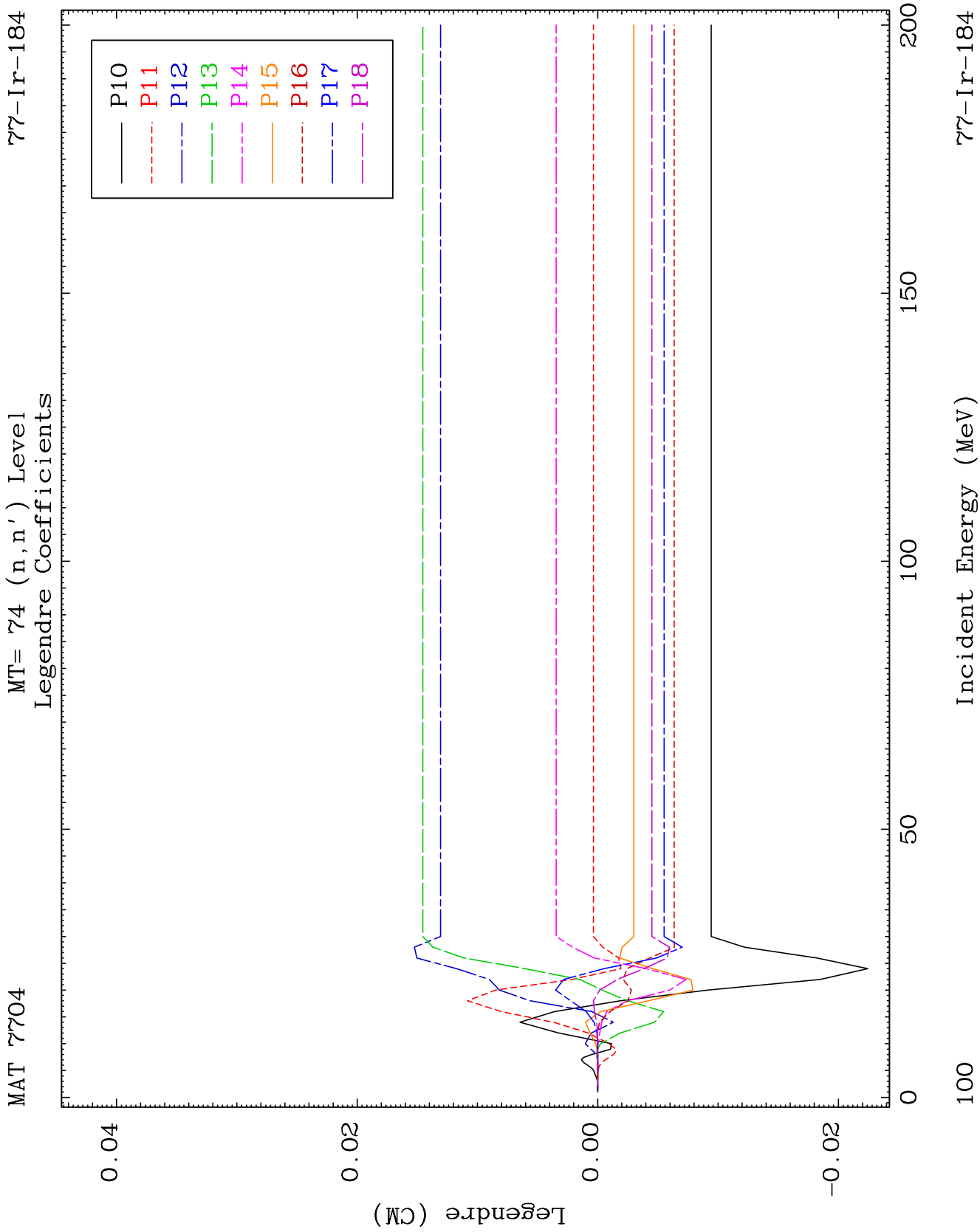
77-Ir-184

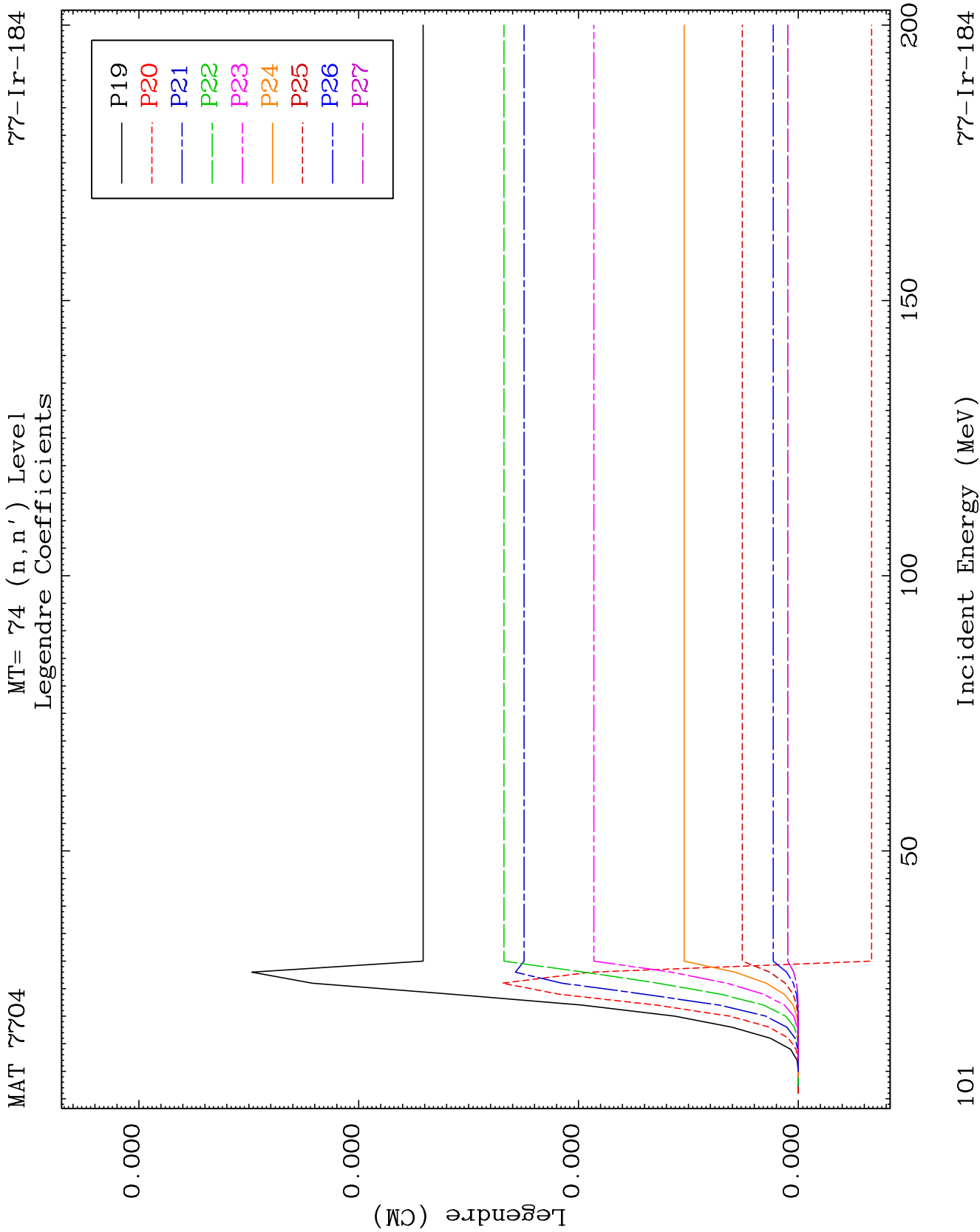


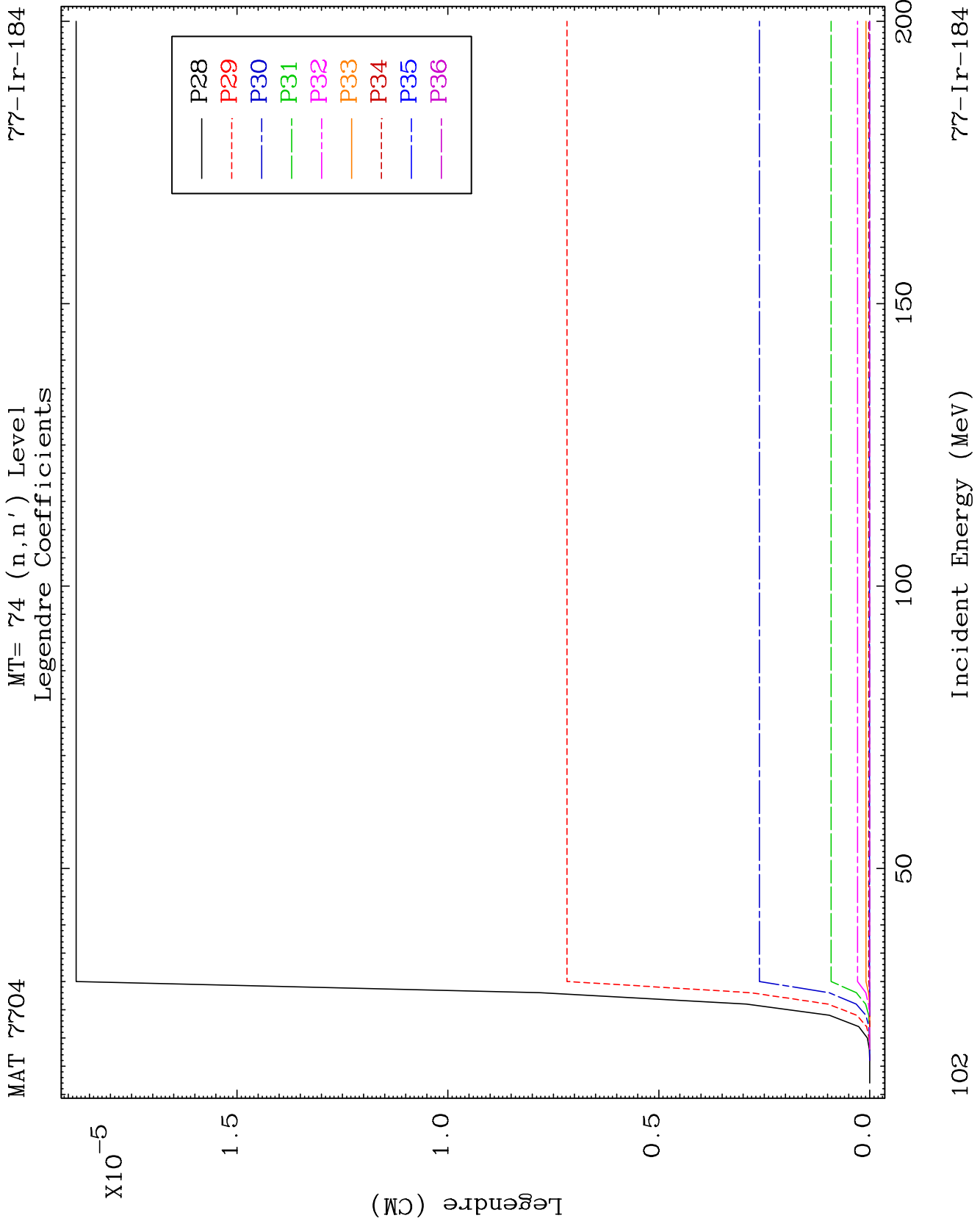
99

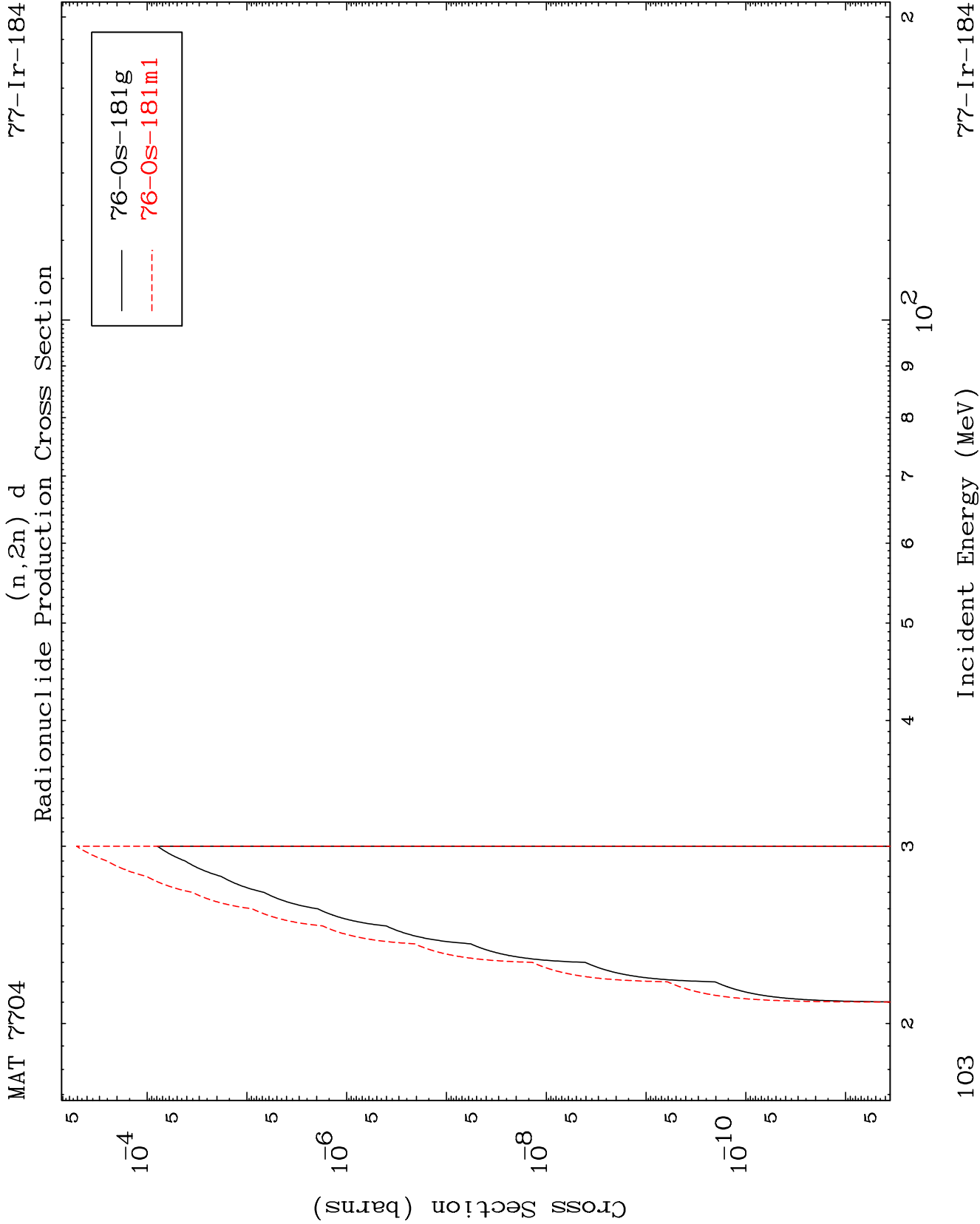
Incident Energy (MeV)

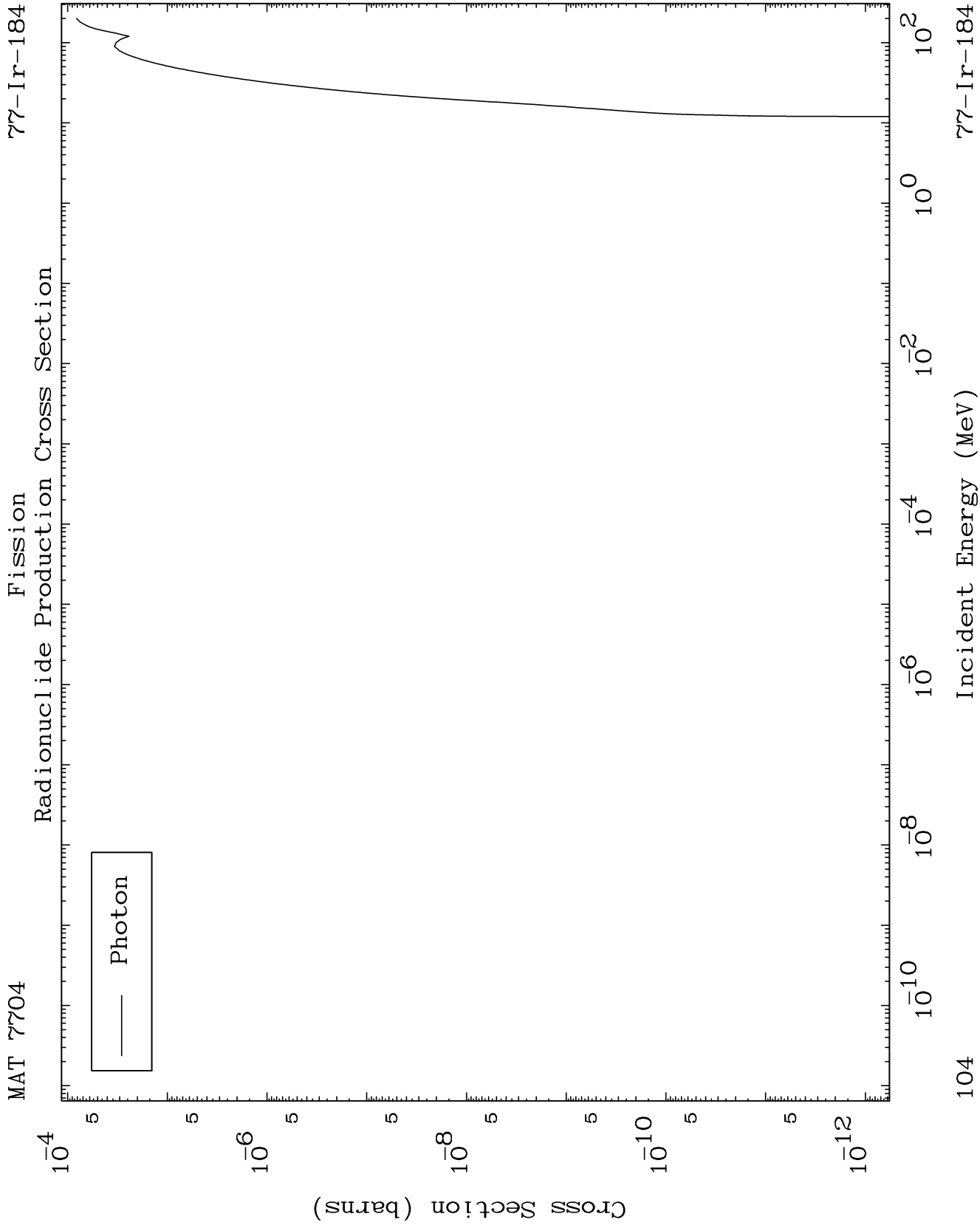
77-1r-184







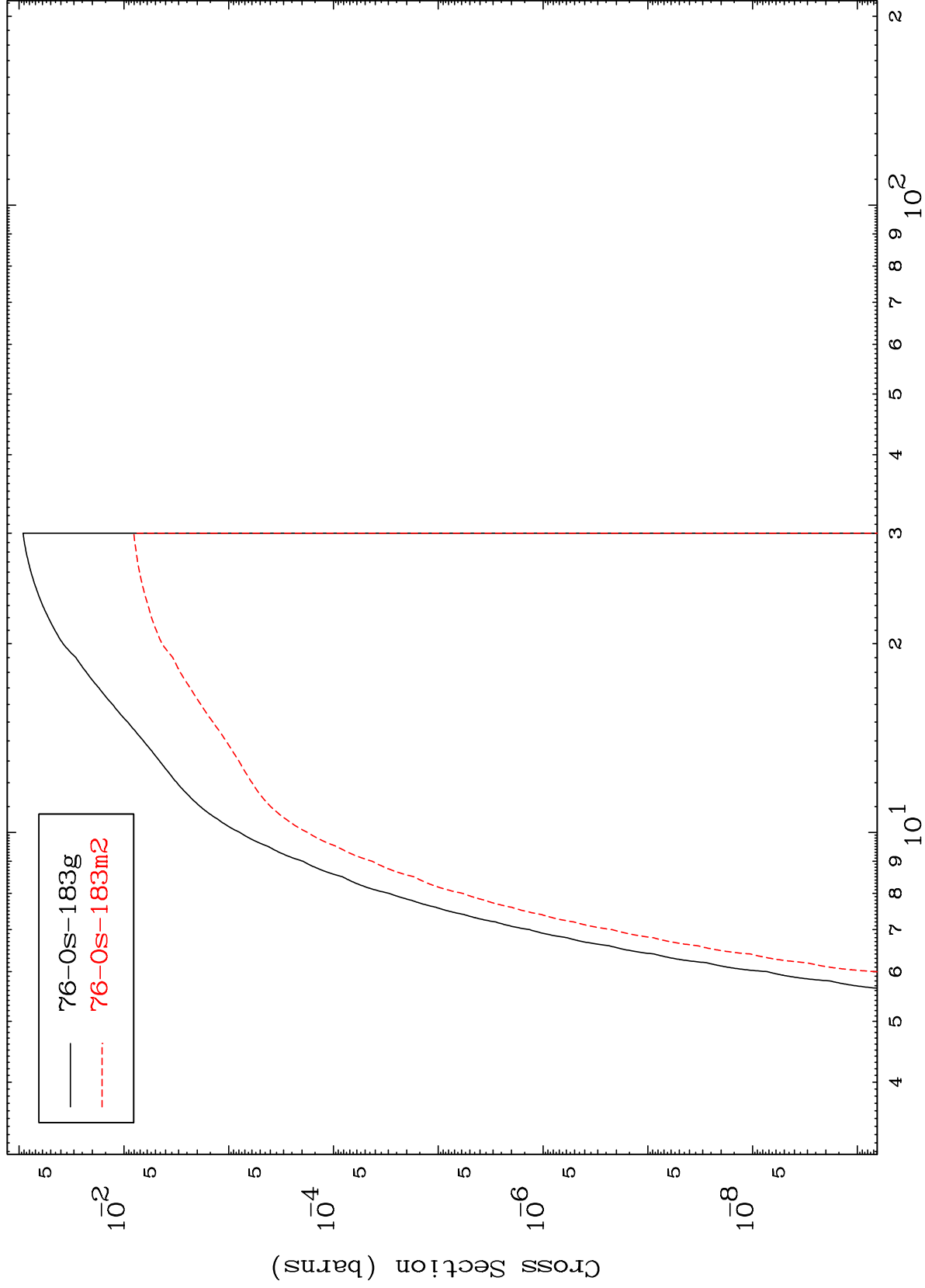




MAT 7704

77-Ir-184

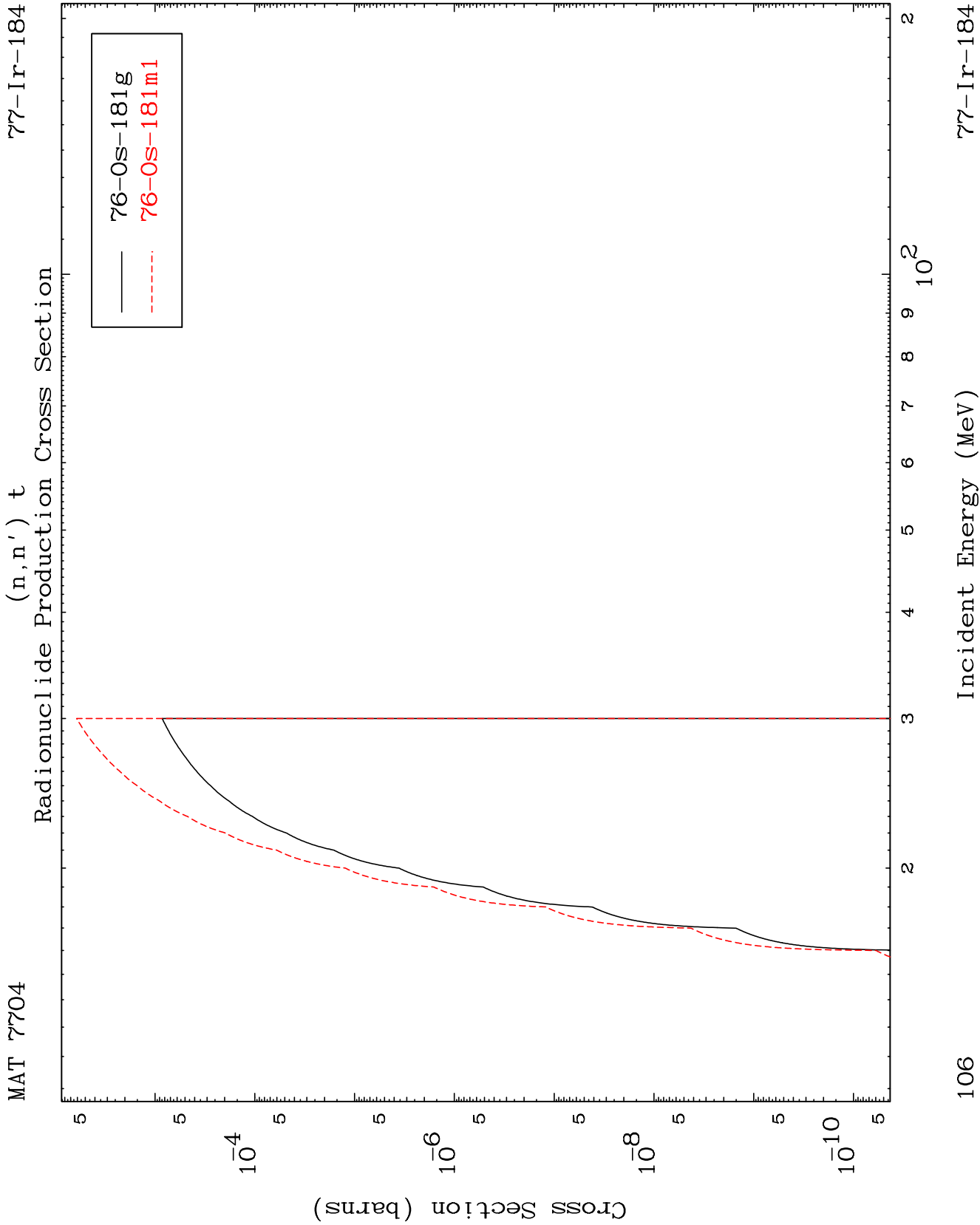
Radionuclide Production Cross Section

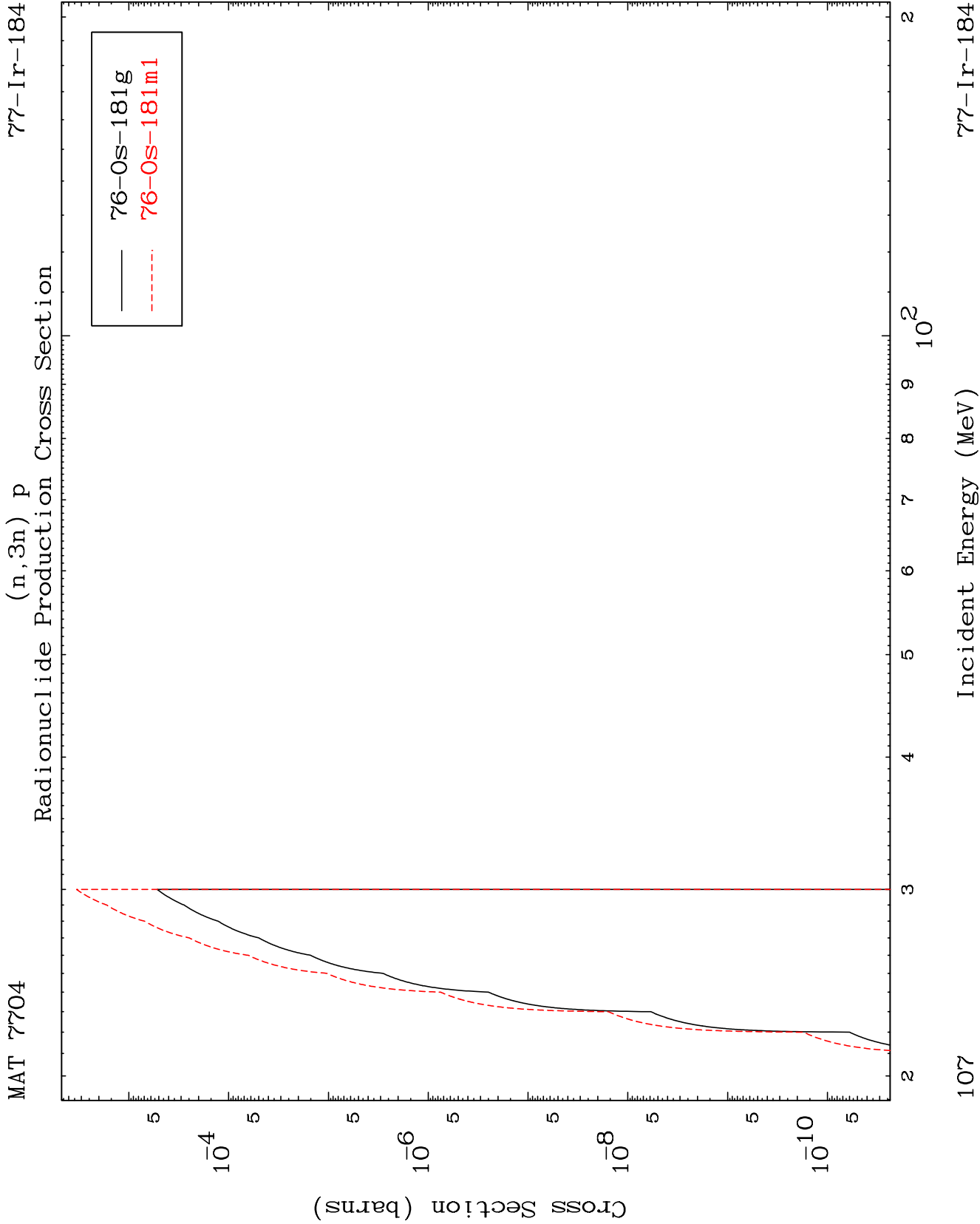


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Incident Energy (MeV)

77-Ir-184

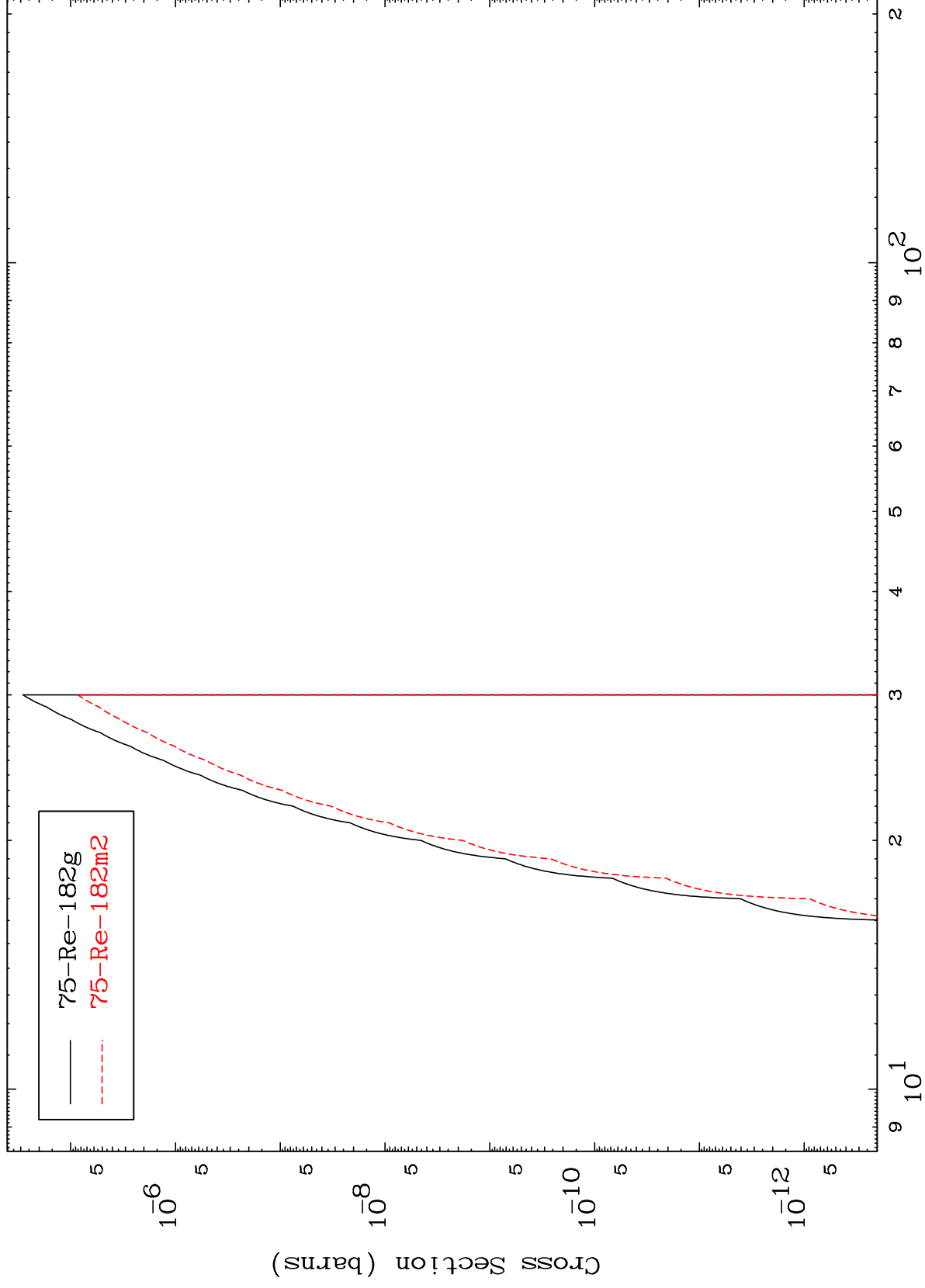




MAT 7704

$^{77}\text{Ir-184}$

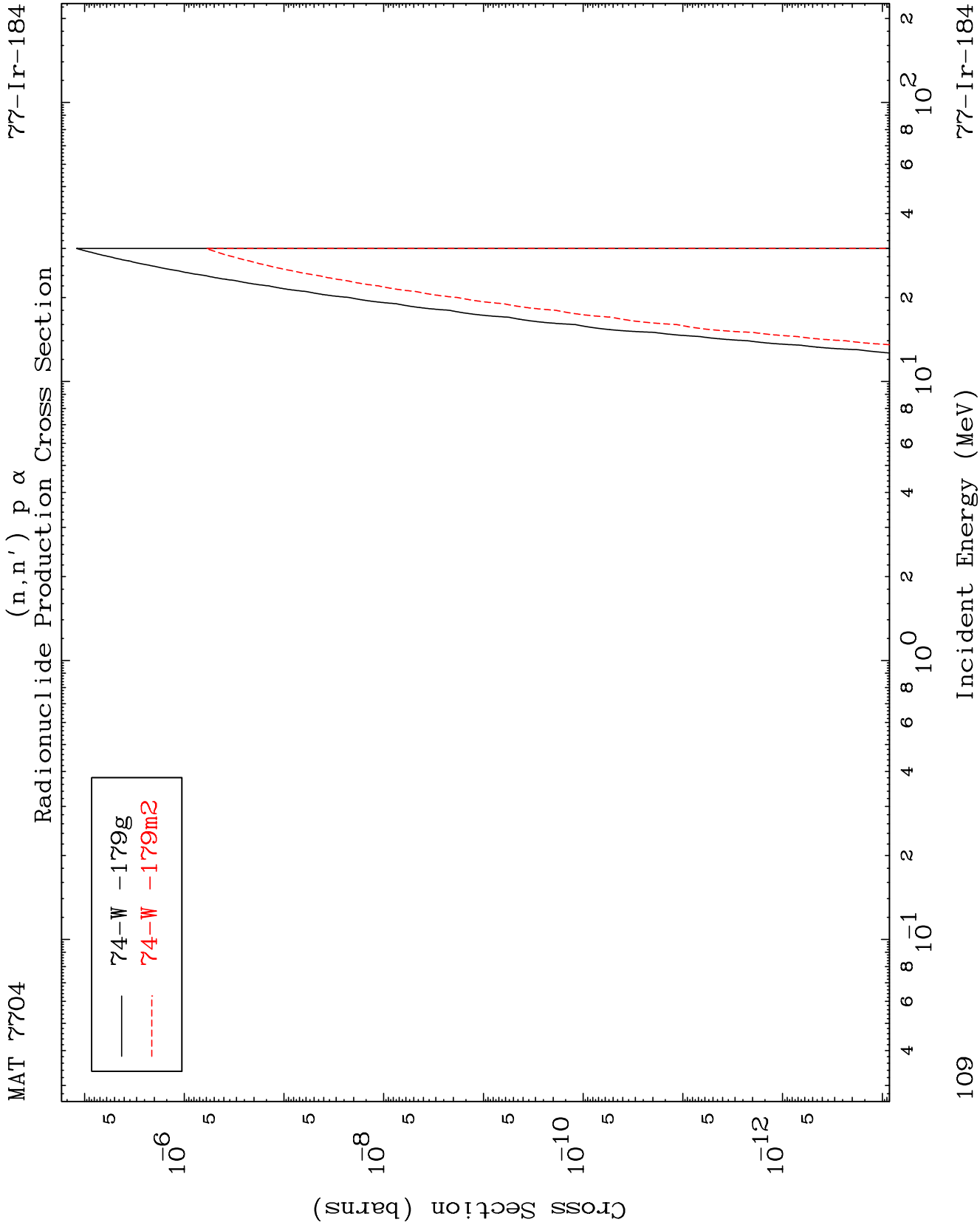
Radionuclide Production Cross Section



$^{77}\text{Ir-184}$

Incident Energy (MeV)

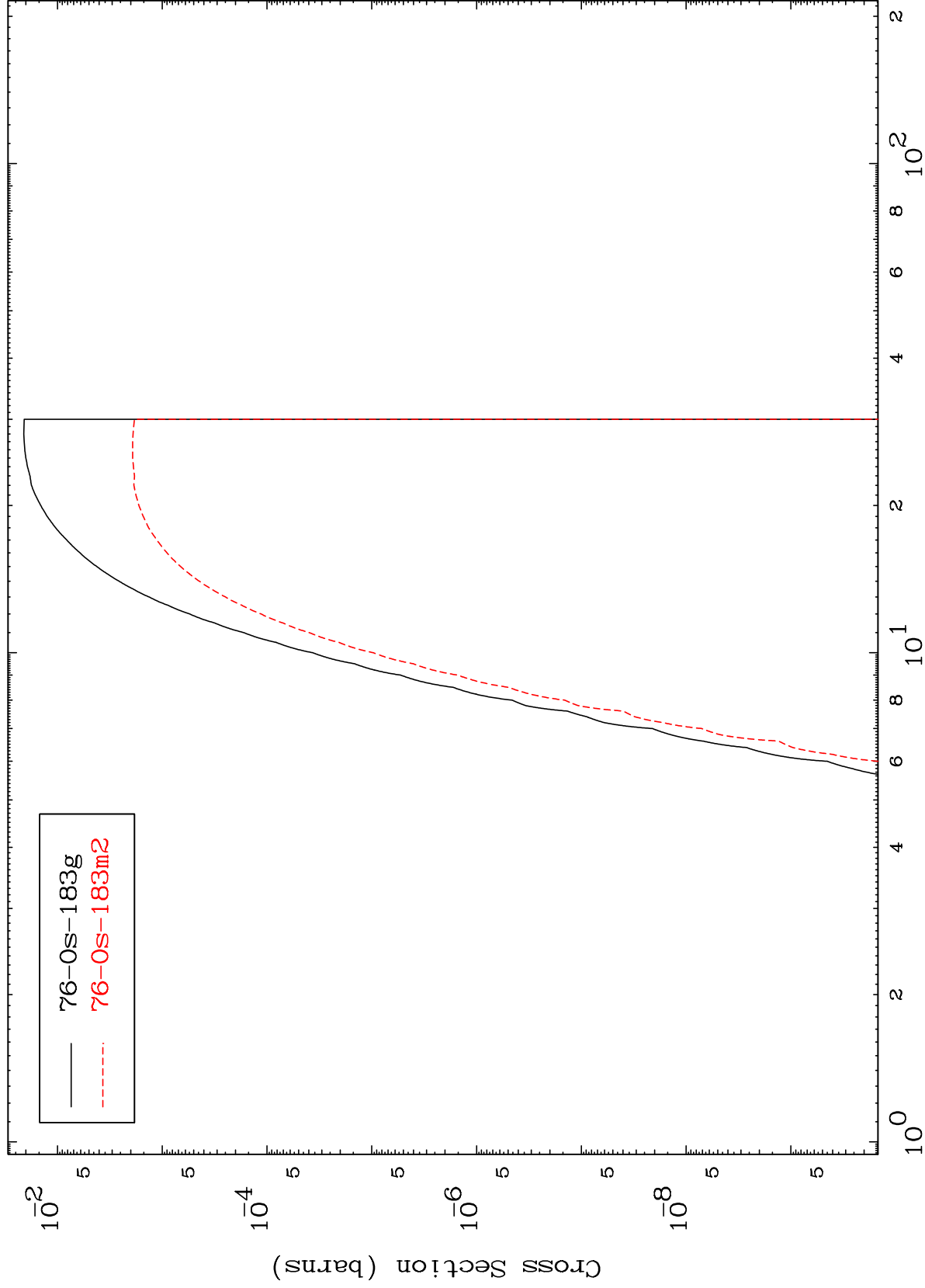
108



MAT 7704

⁷⁷Ir-184

Radionuclide Production Cross Section



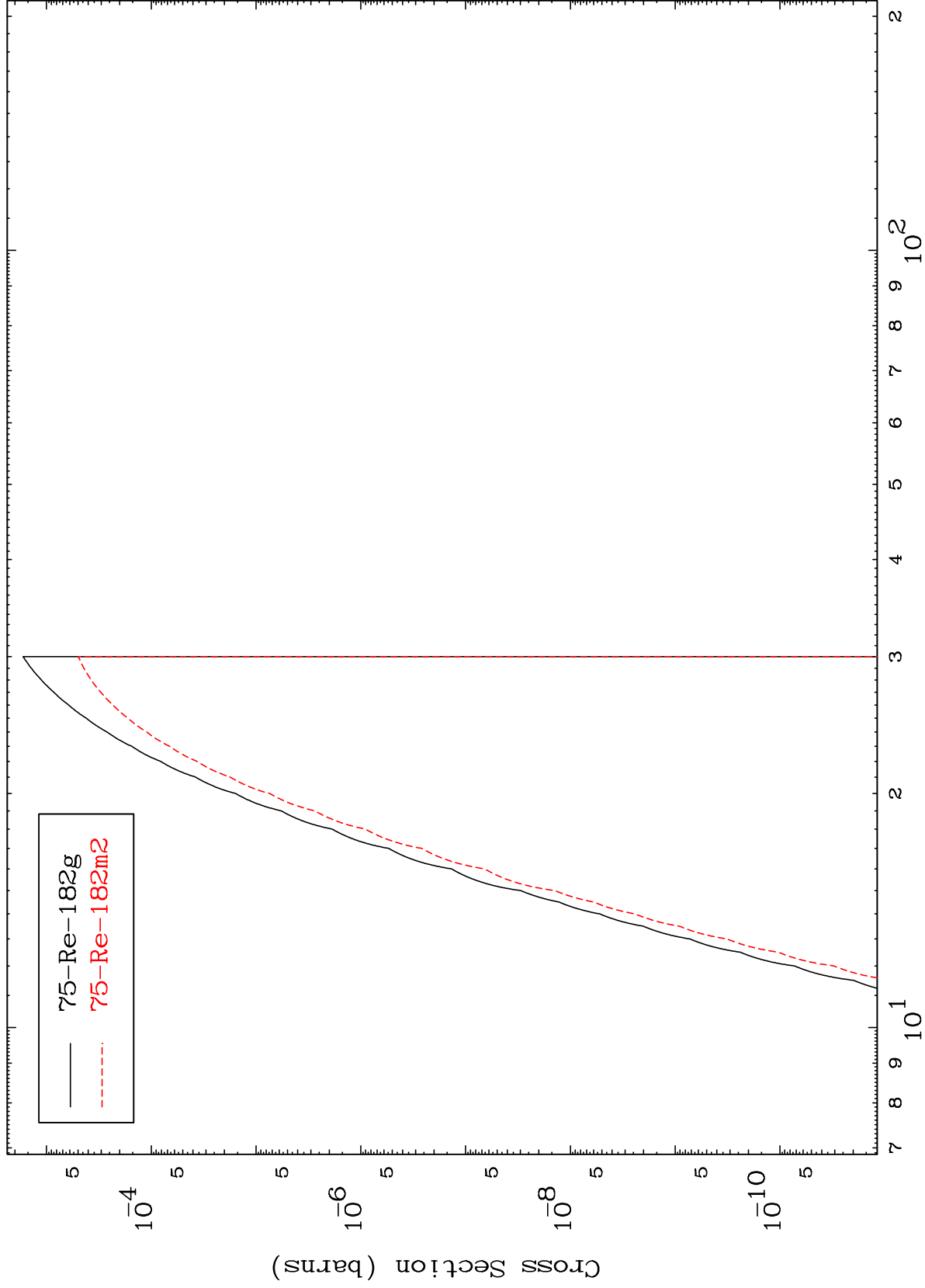
⁷⁷Ir-184

Incident Energy (MeV)

MAT 7704

⁷⁷Ir-184

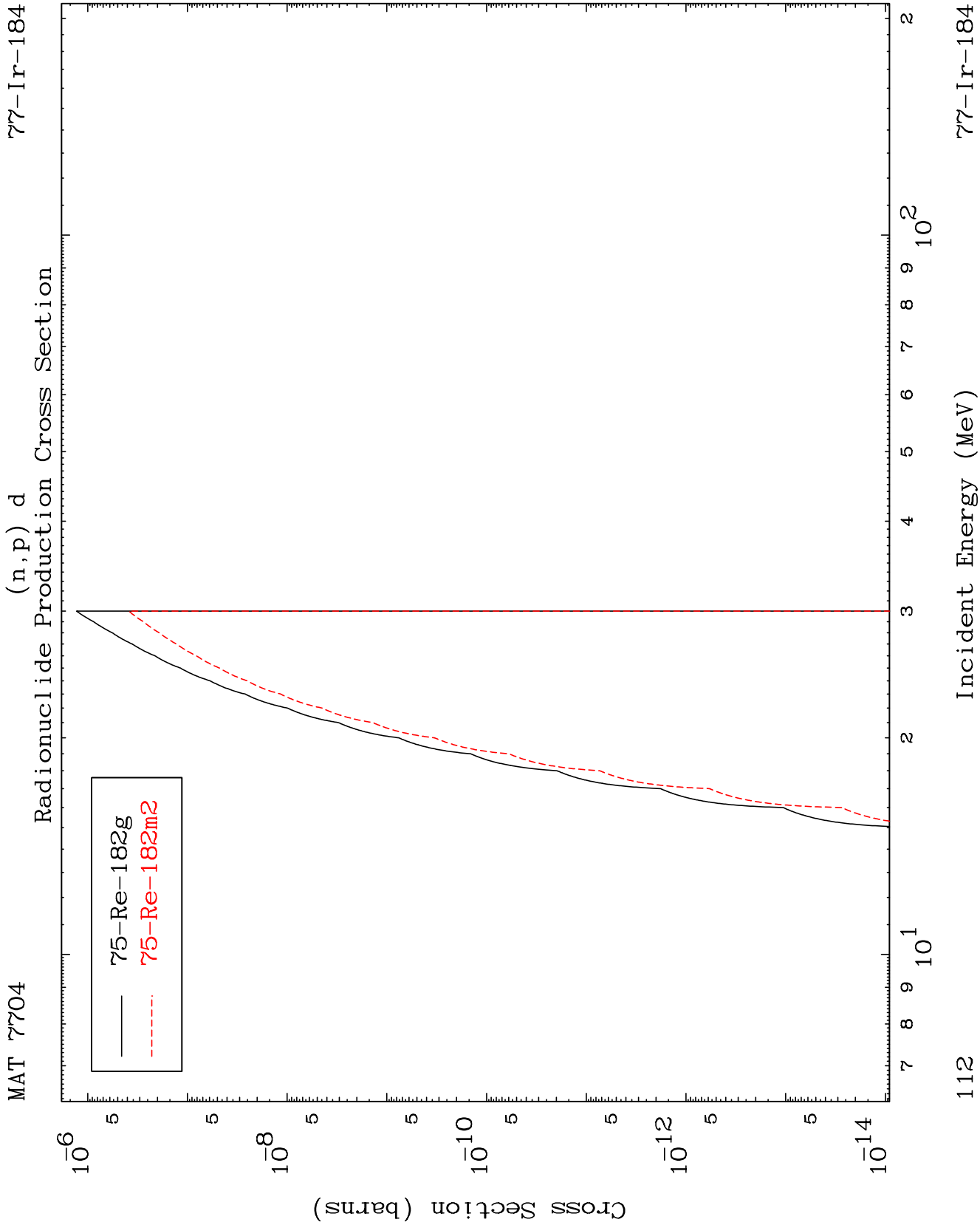
Radionuclide Production Cross Section



⁷⁷Ir-184

Incident Energy (MeV)

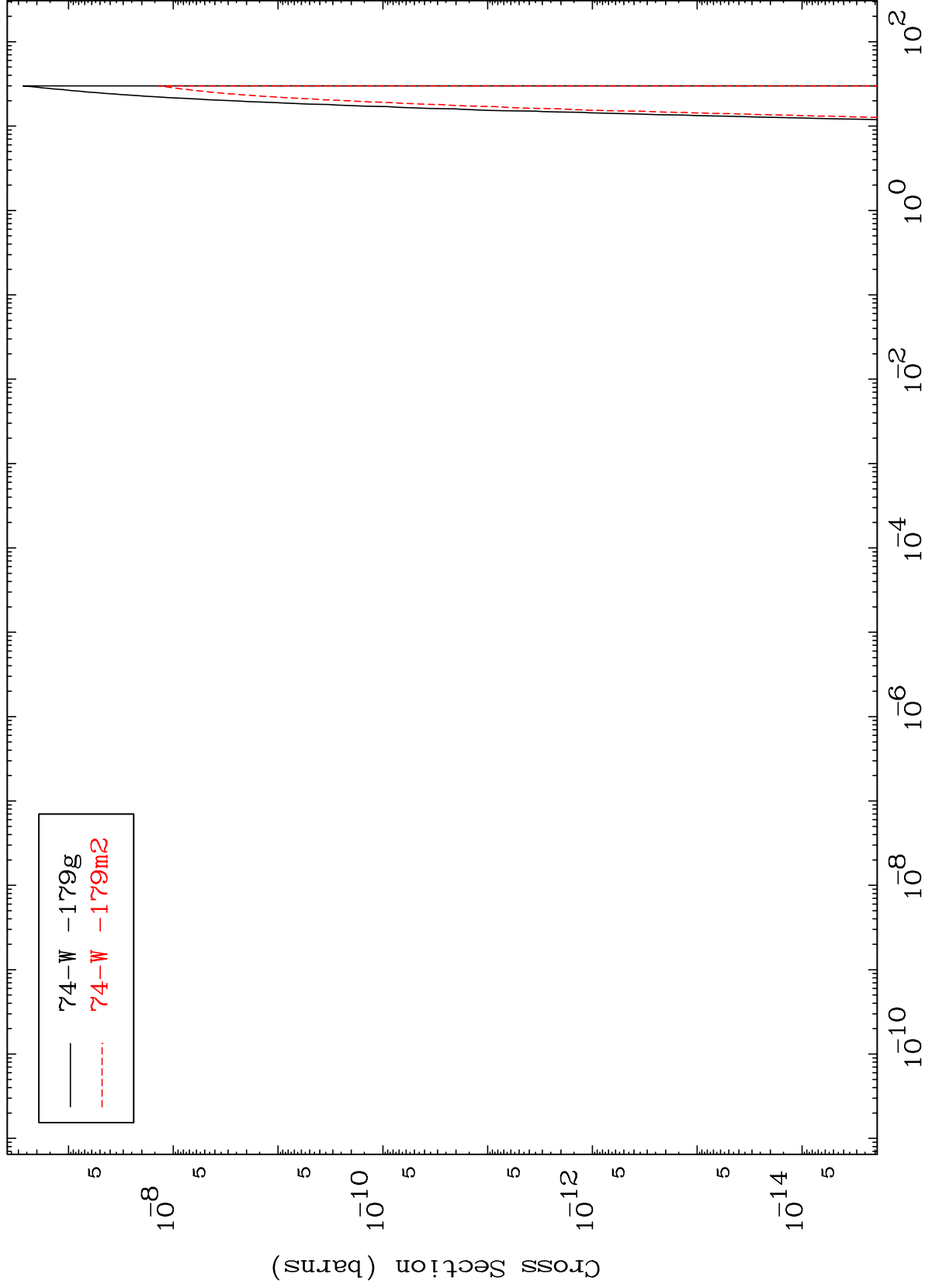
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MAT 7704

$^{77}\text{Ir-184}$

$(n,d) \alpha$
Radionuclide Production Cross Section



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$^{77}\text{Ir-184}$