

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

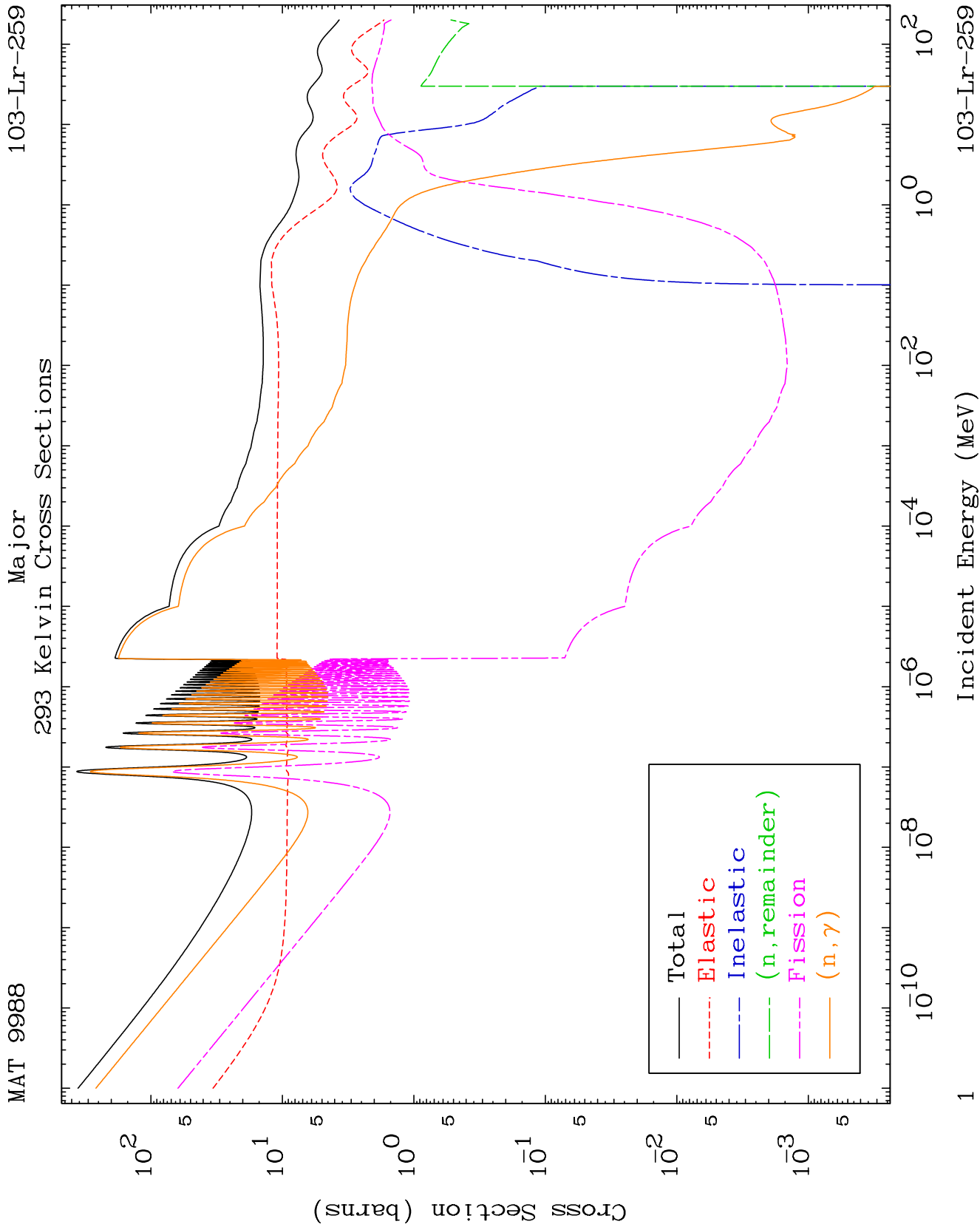
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

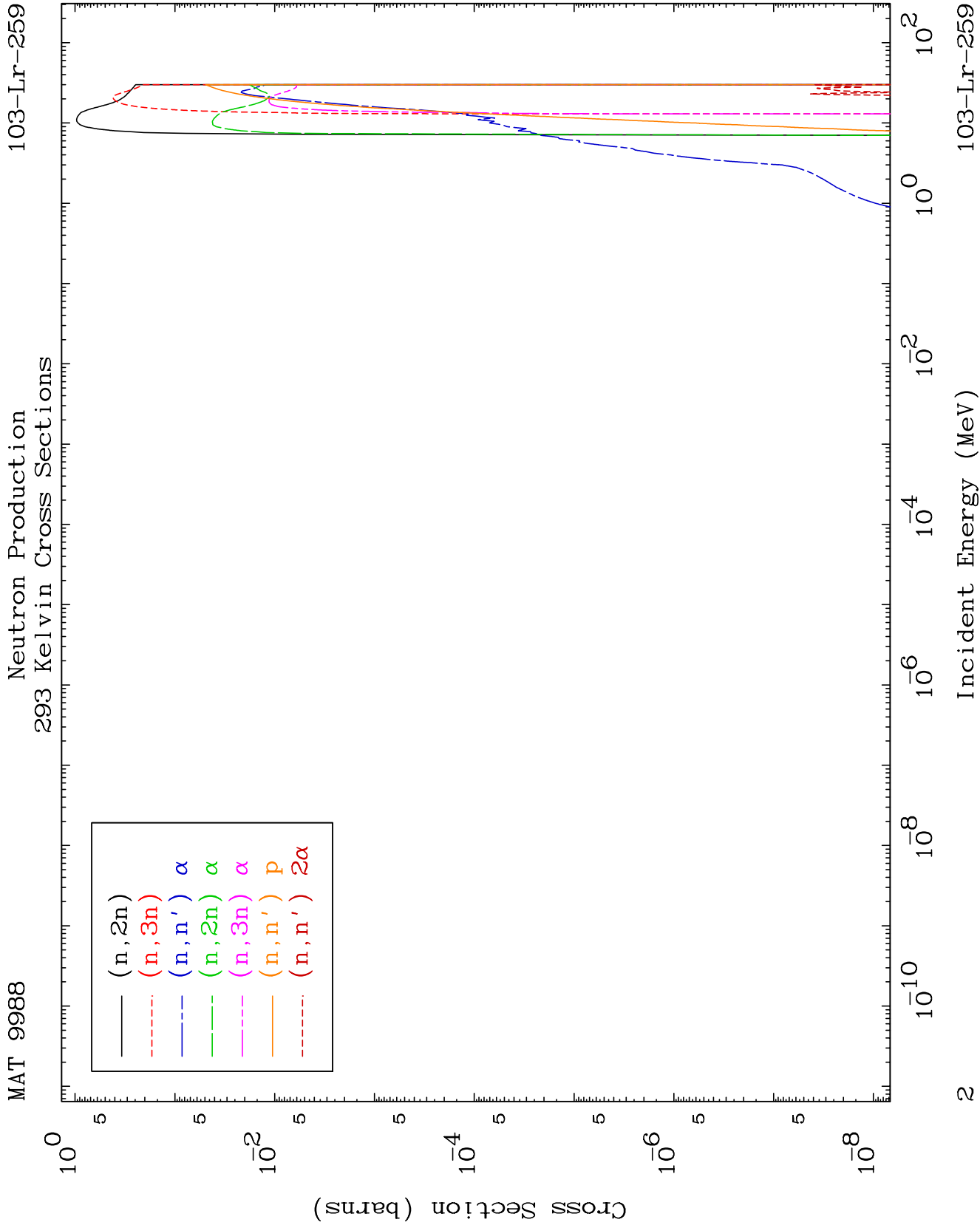
Tele: 925-443-1911

E.Mail: redcullen1@comcast.net

Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

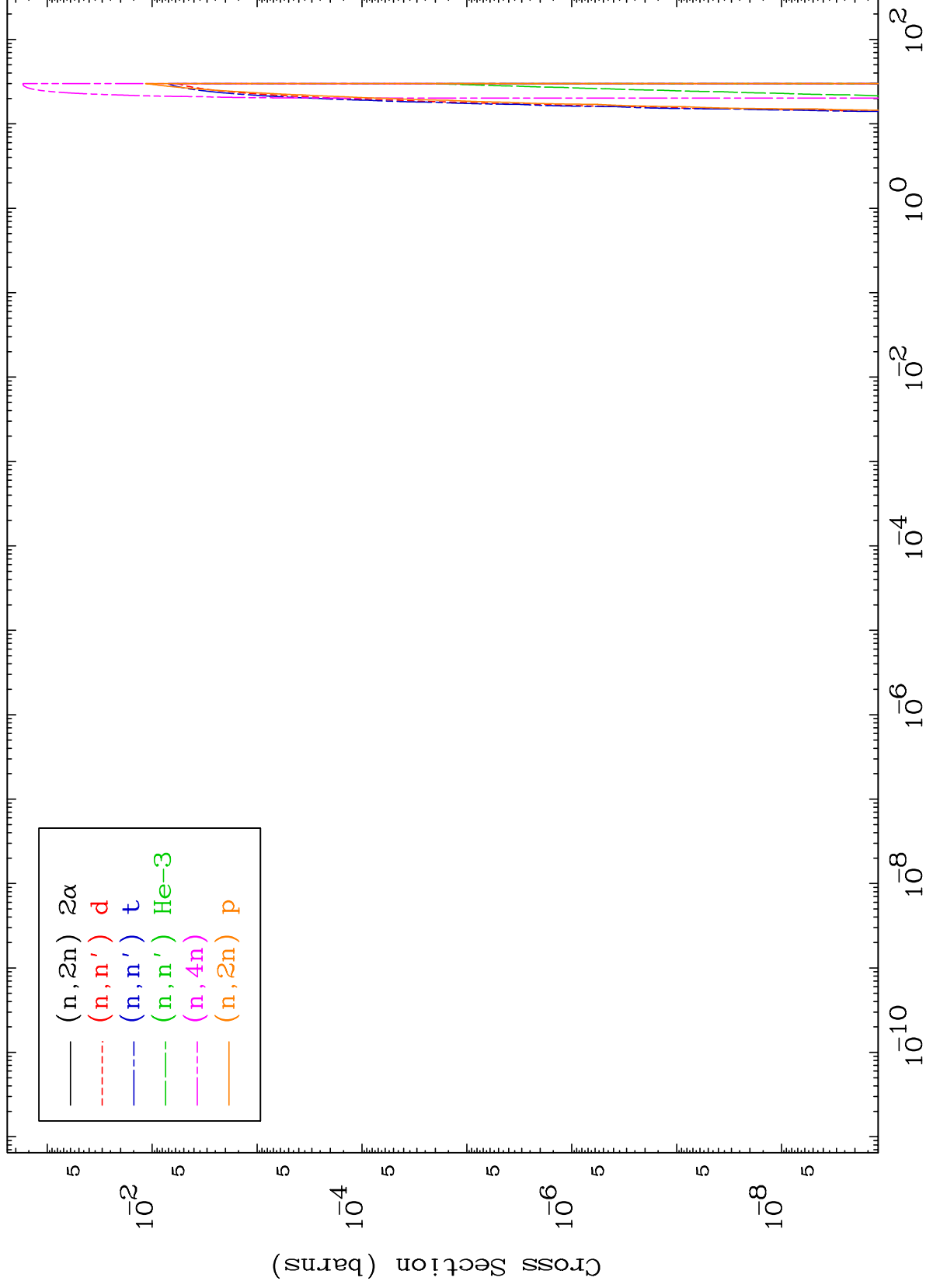


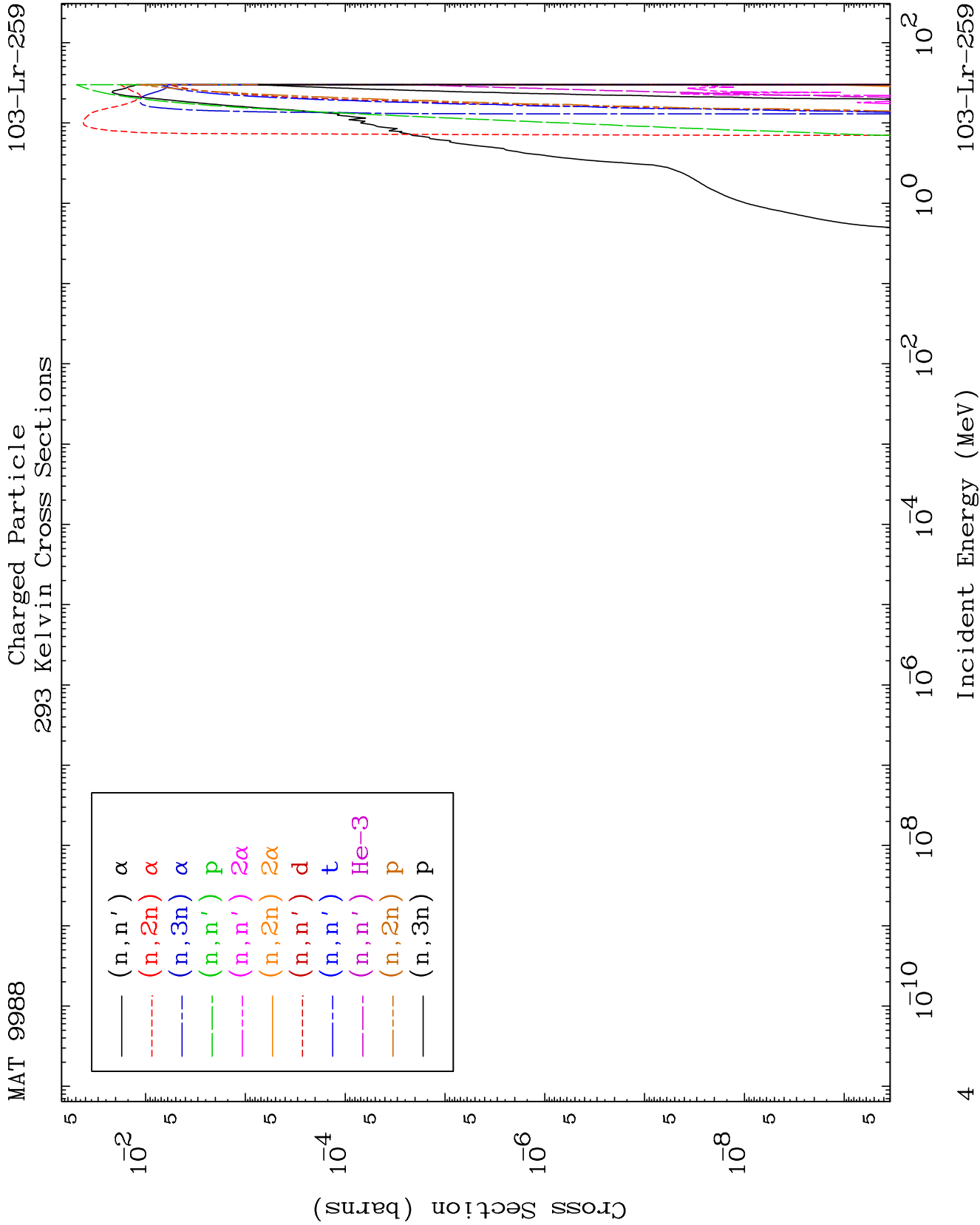


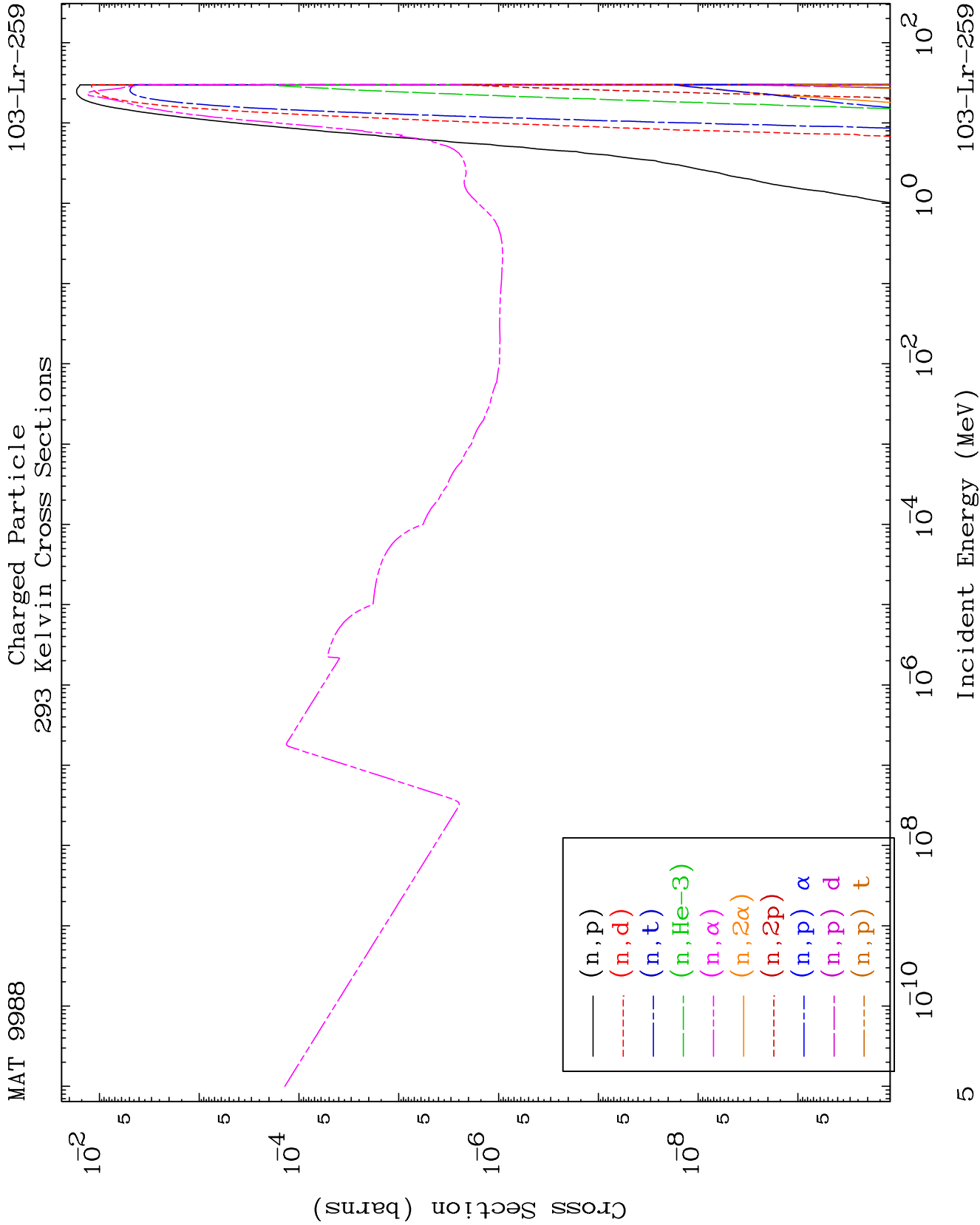
MAT 9988

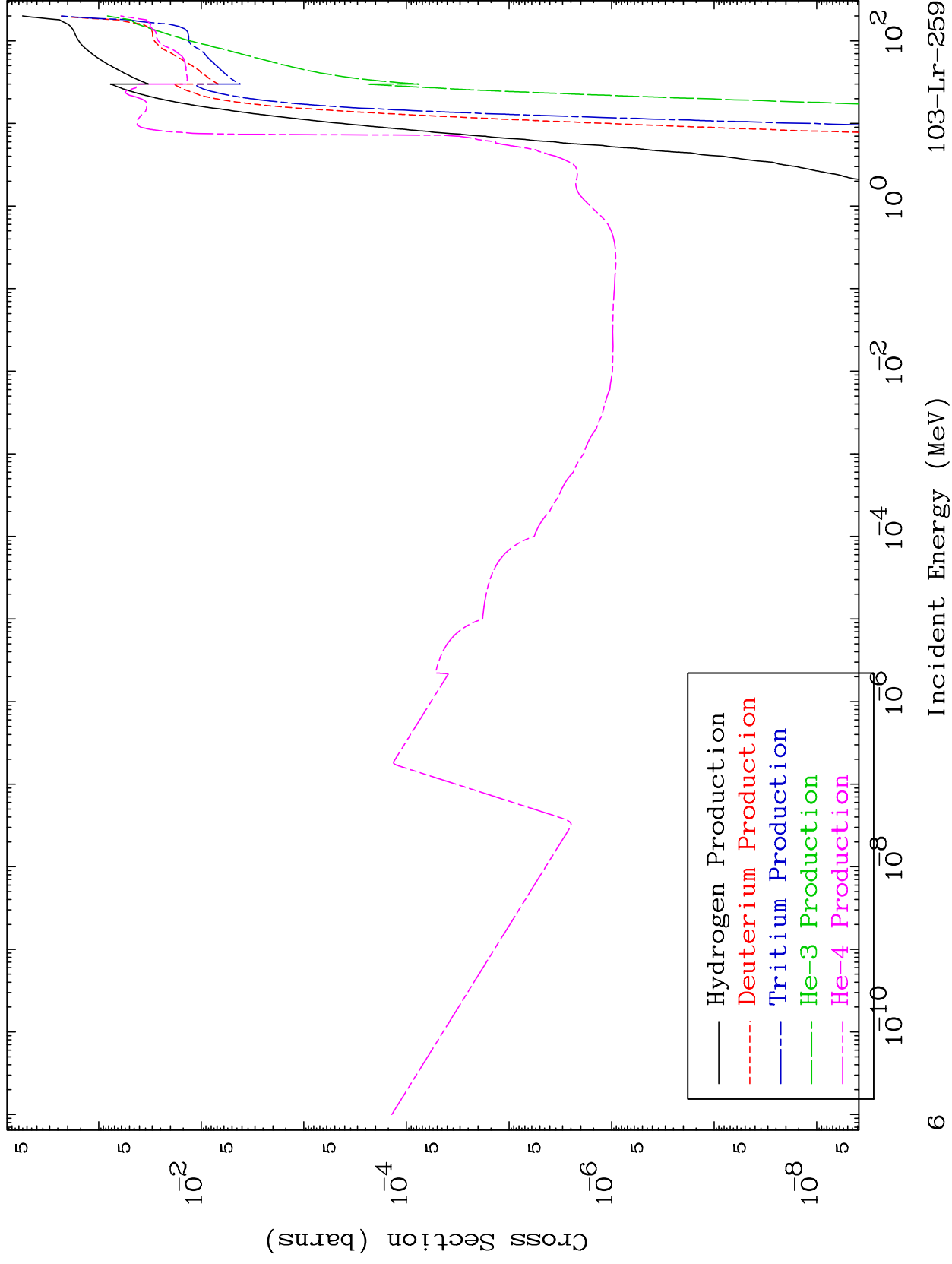
Neutron Production
293 Kelvin Cross Sections

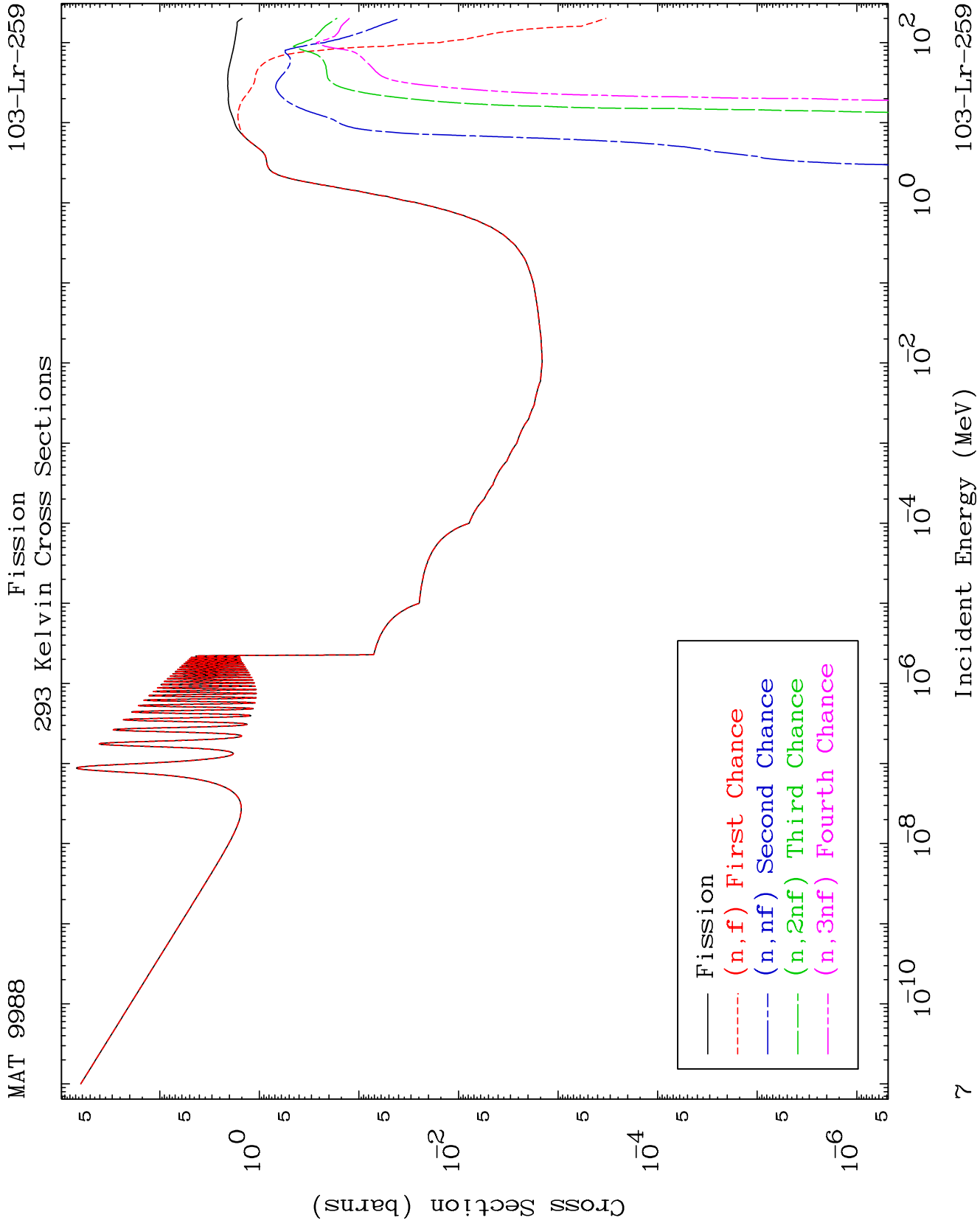
103-Lr-259







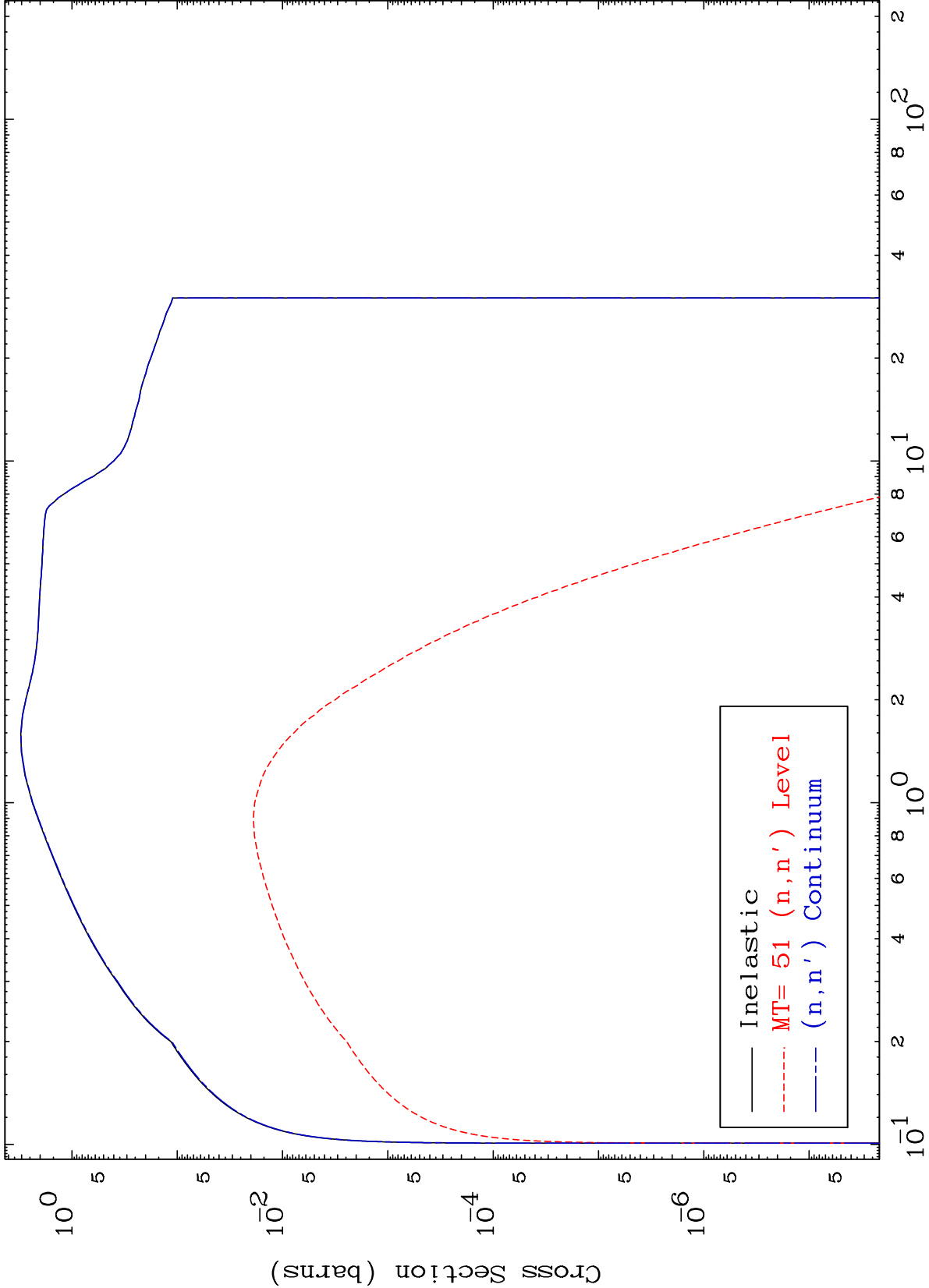




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(n,n') Level
293 Kelvin Cross Sections

103-Lr-259



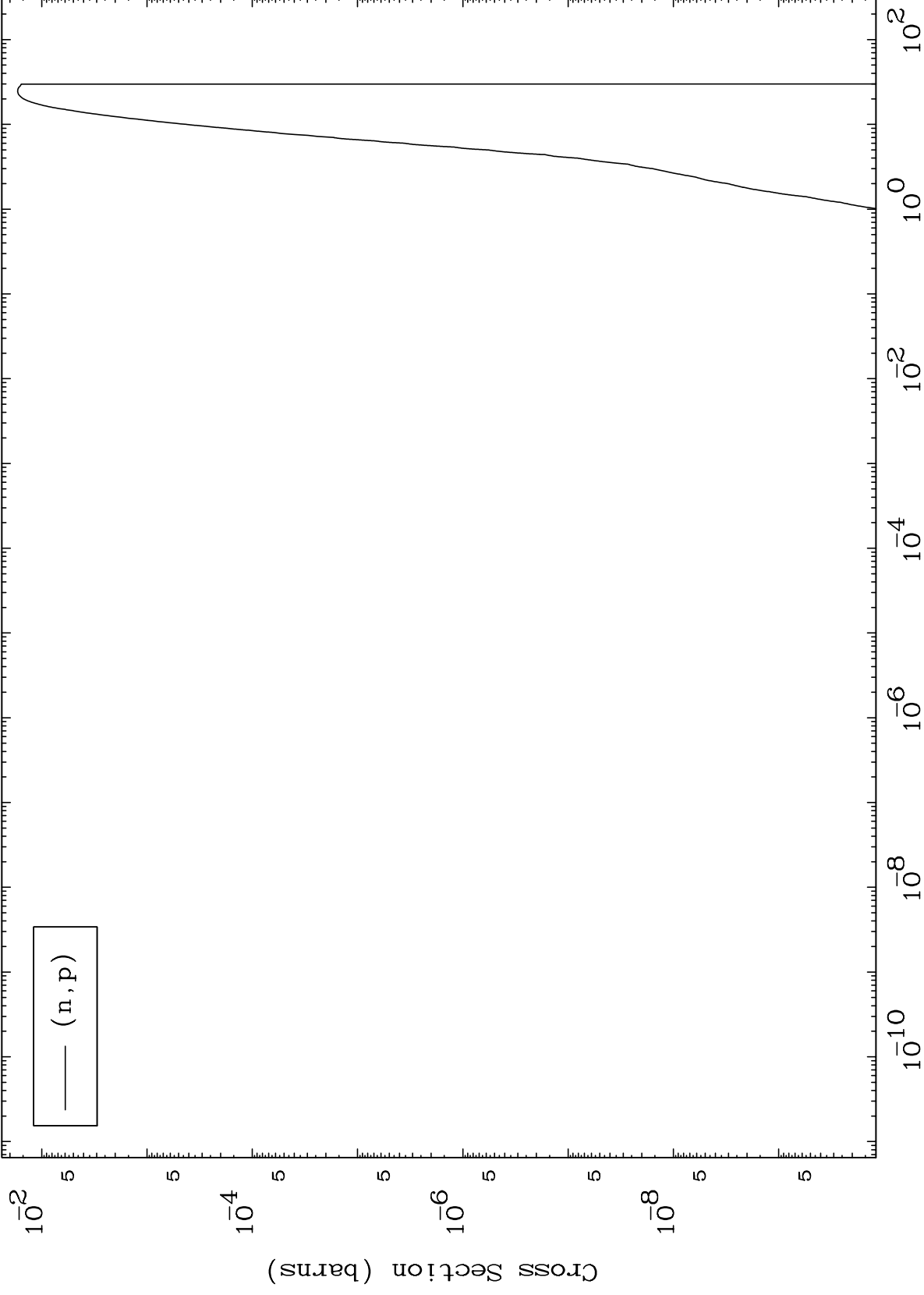
Incident Energy (MeV)

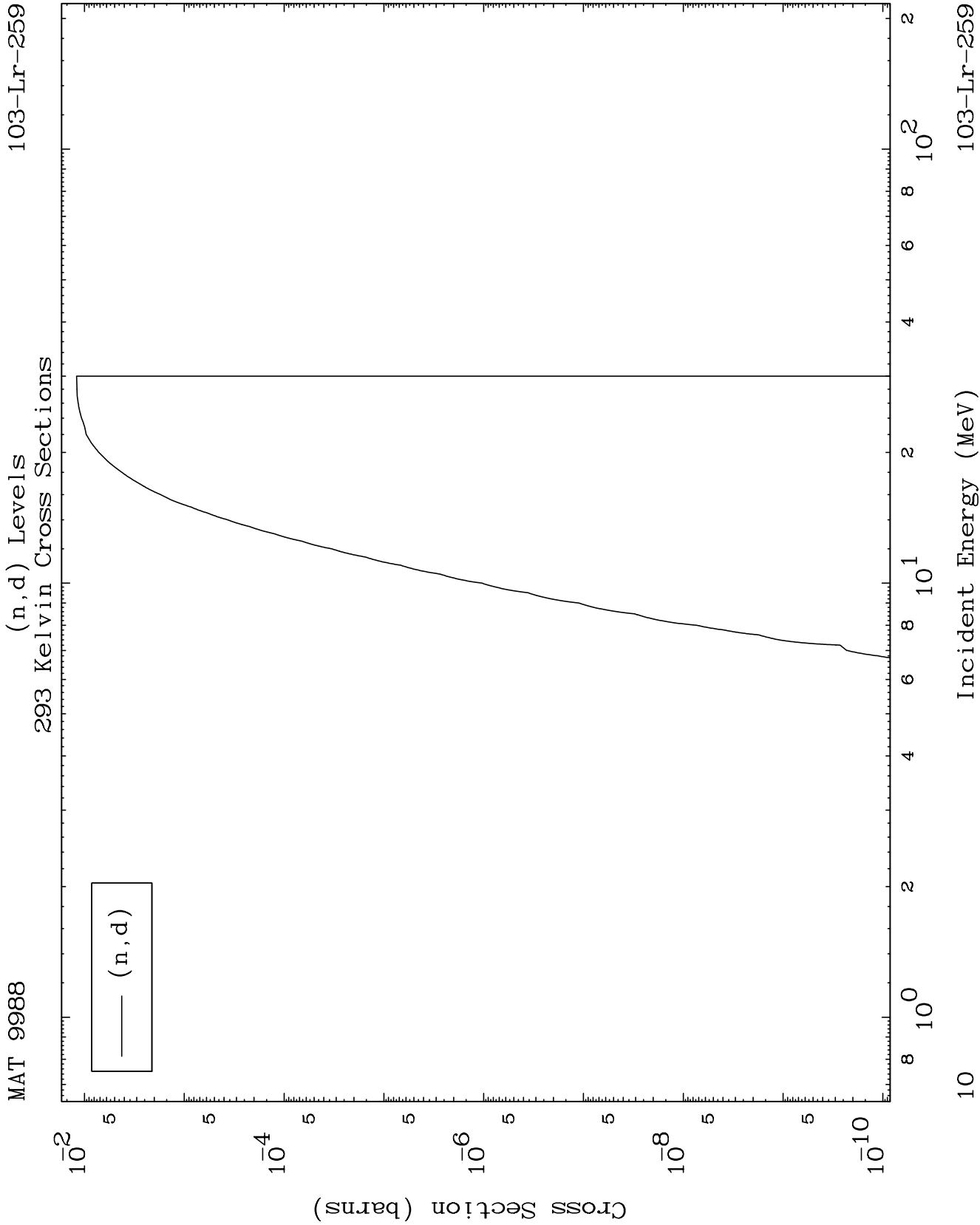
103-Lr-259

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(n,p) Levels
293 Kelvin Cross Sections

103-Lr-259

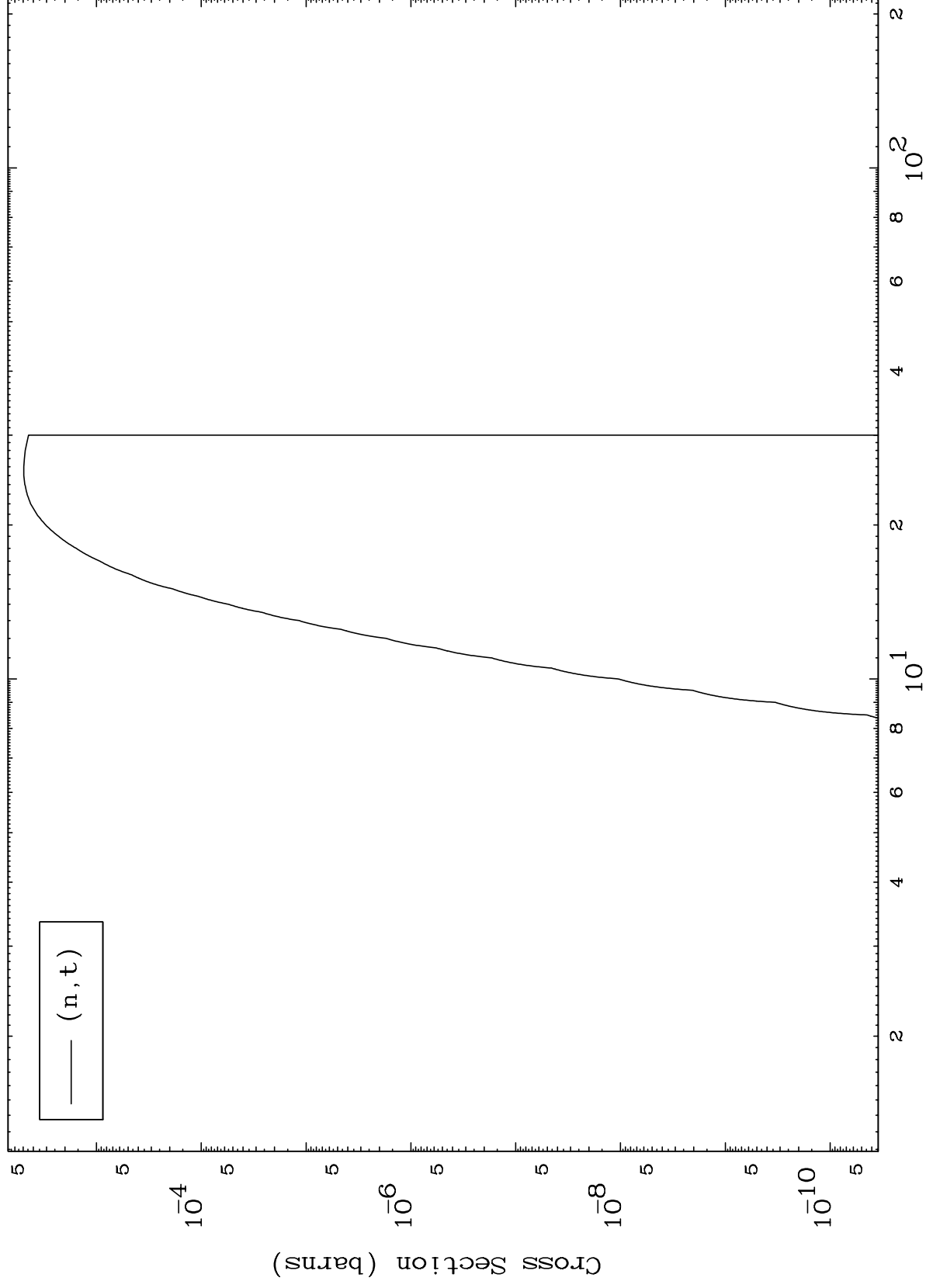




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103-Lr-259

(n,t) Levels
293 Kelvin Cross Sections



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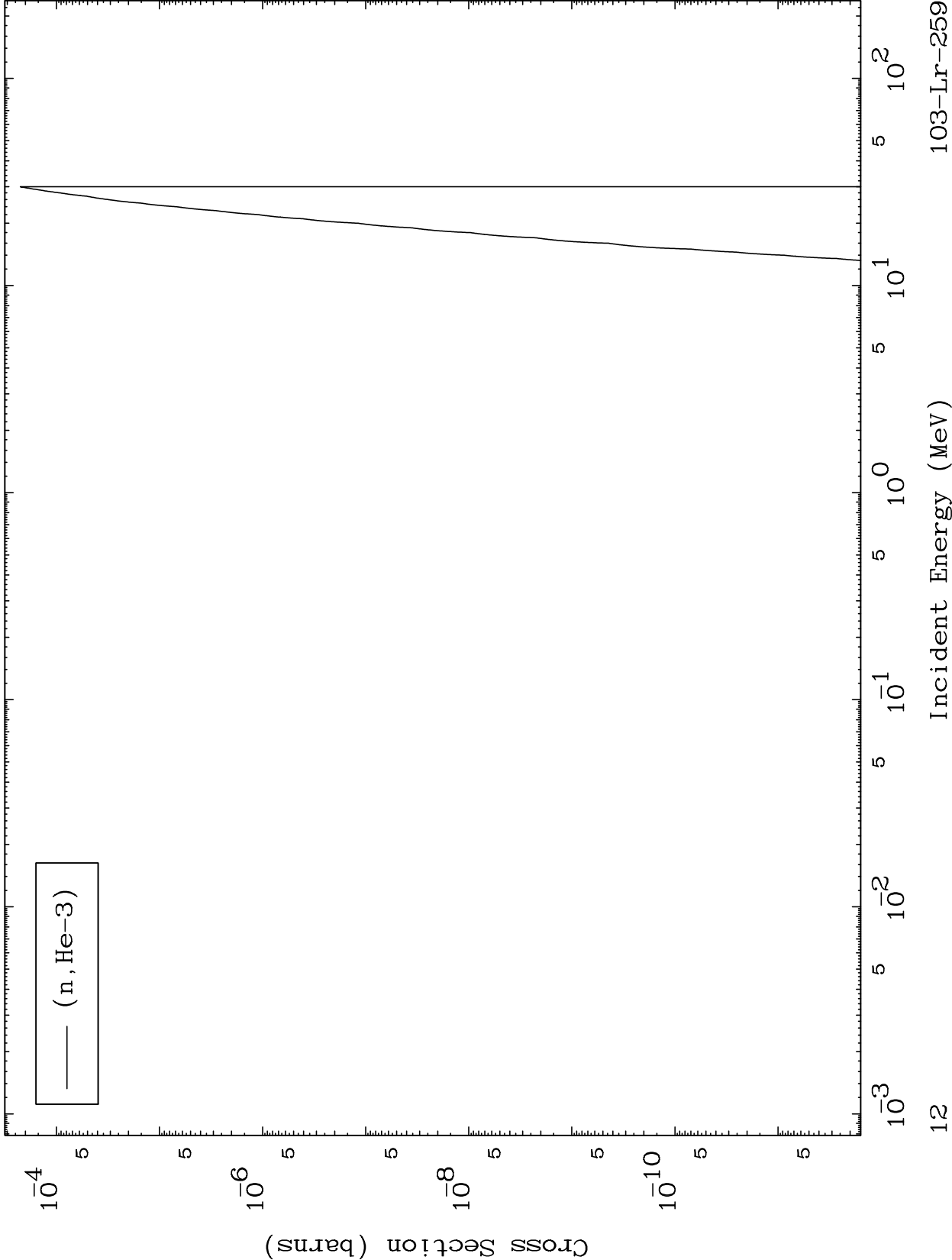
Incident Energy (MeV)

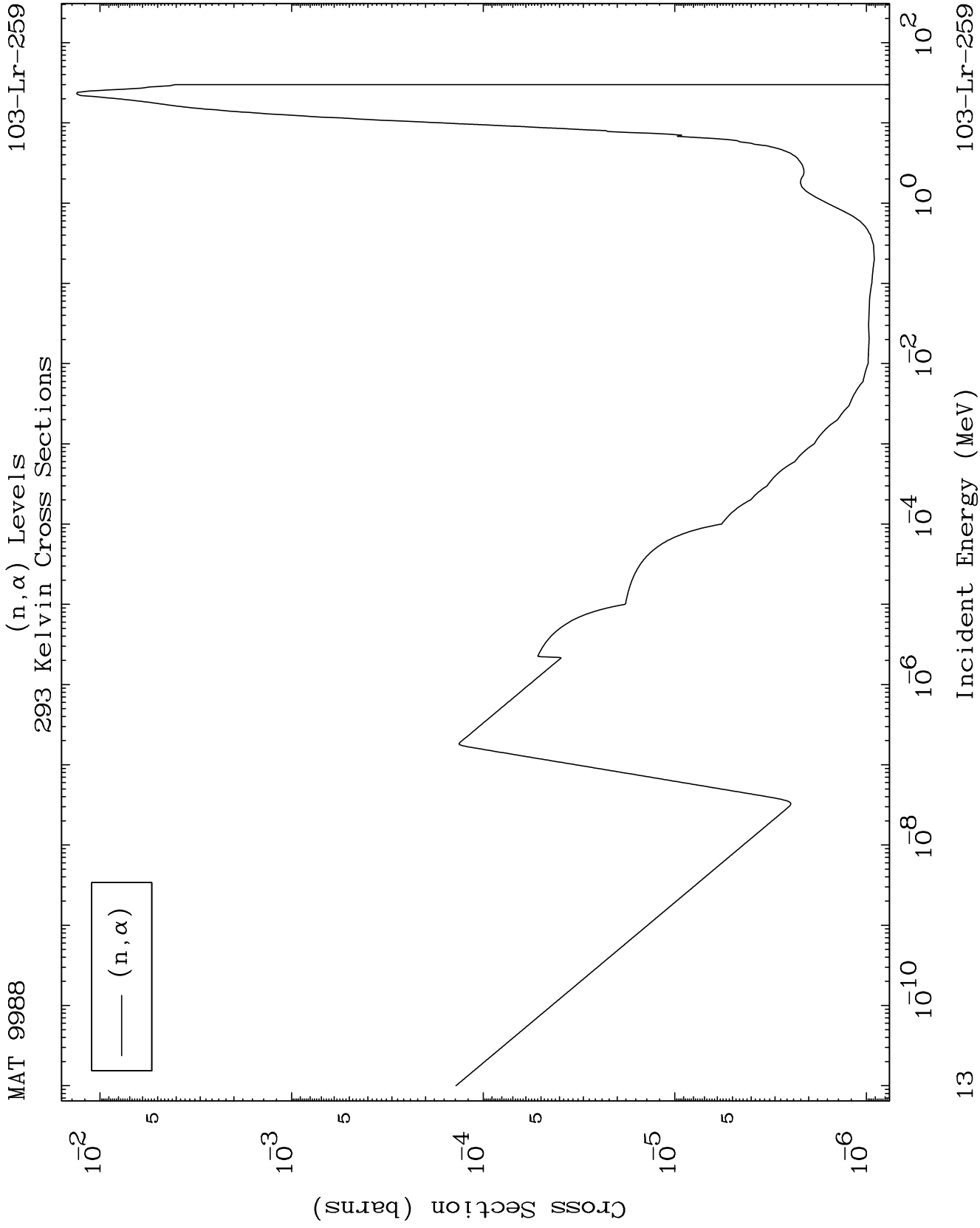
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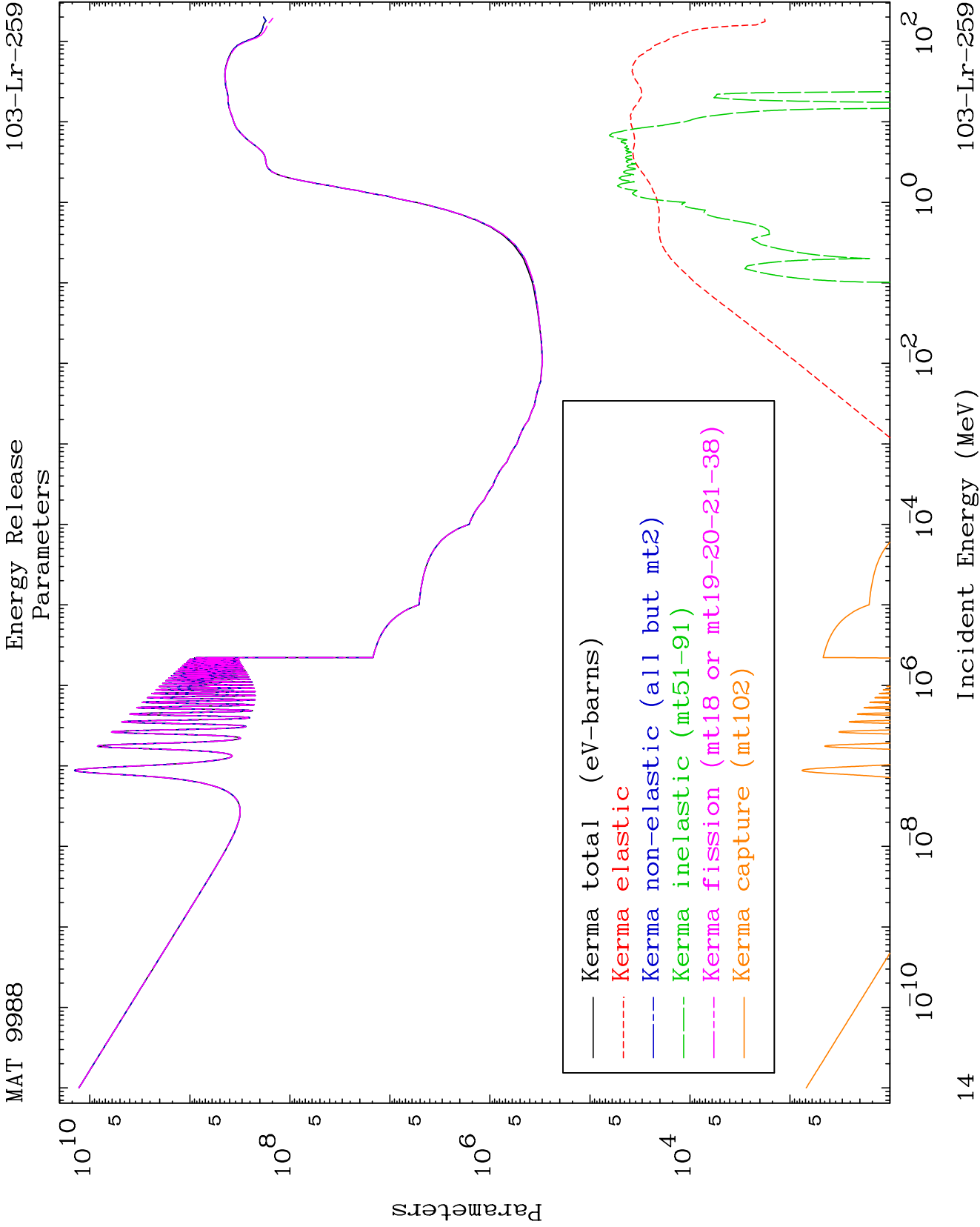
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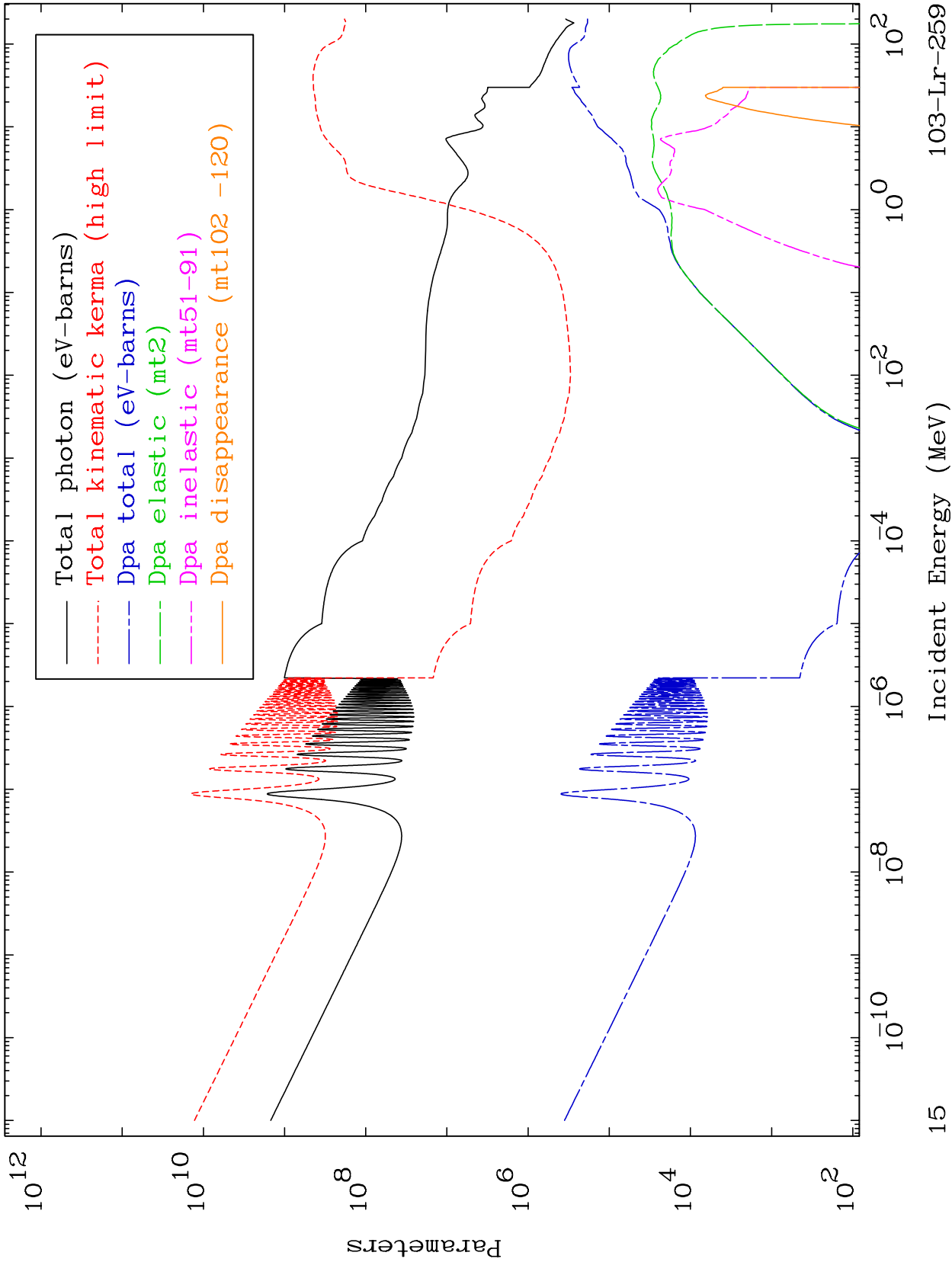
(n,He3) Levels
293 Kelvin Cross Sections

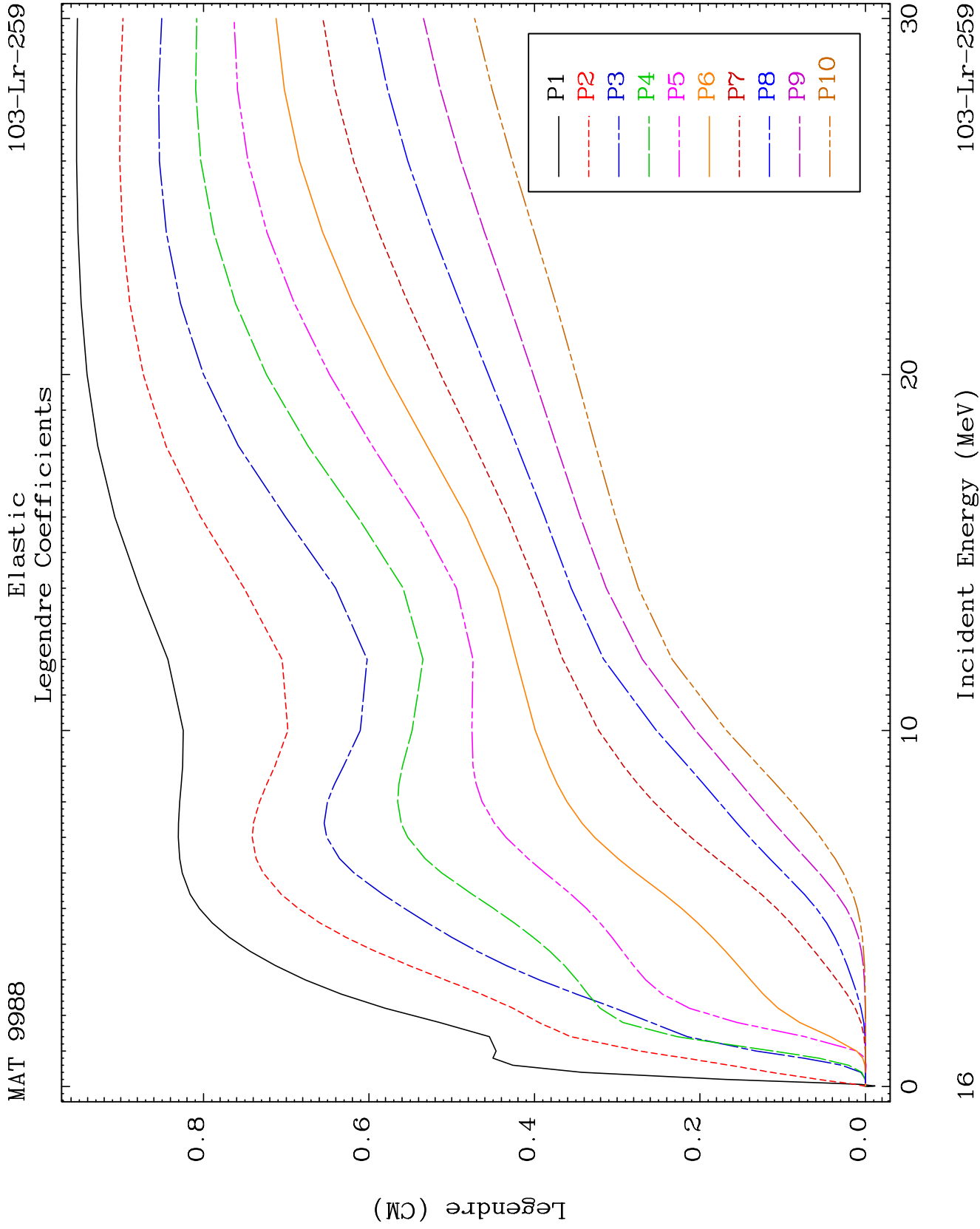
103-Lr-259

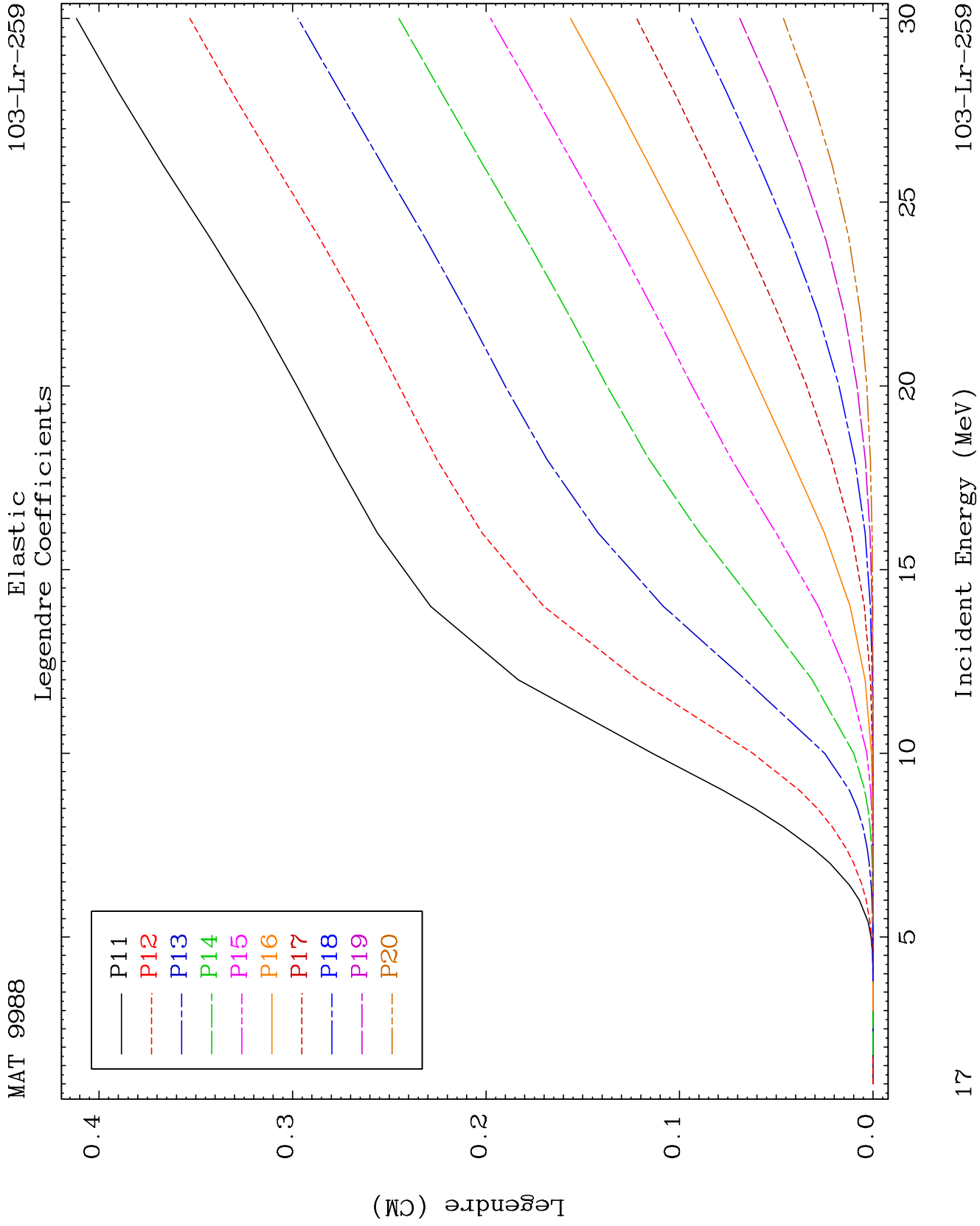


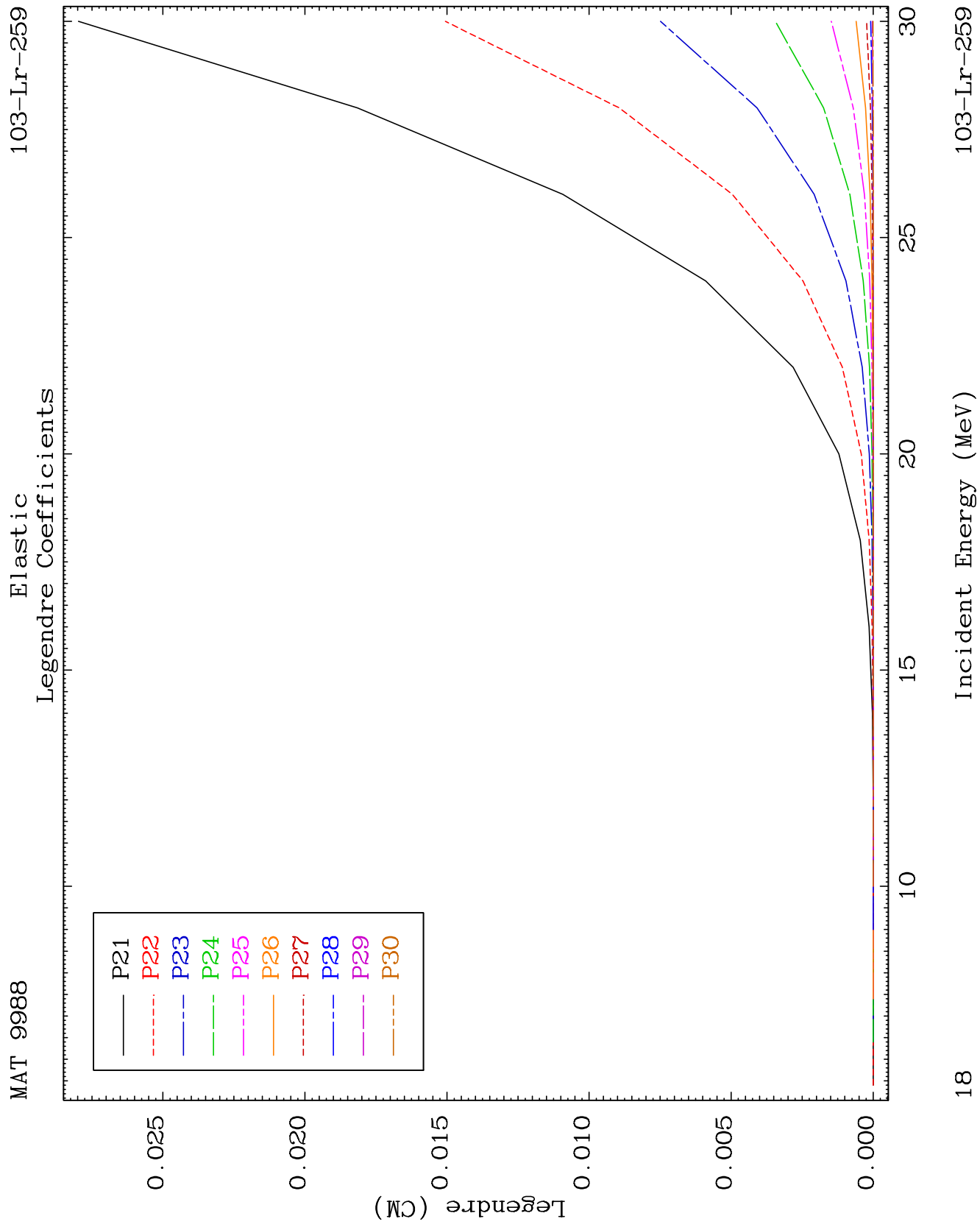


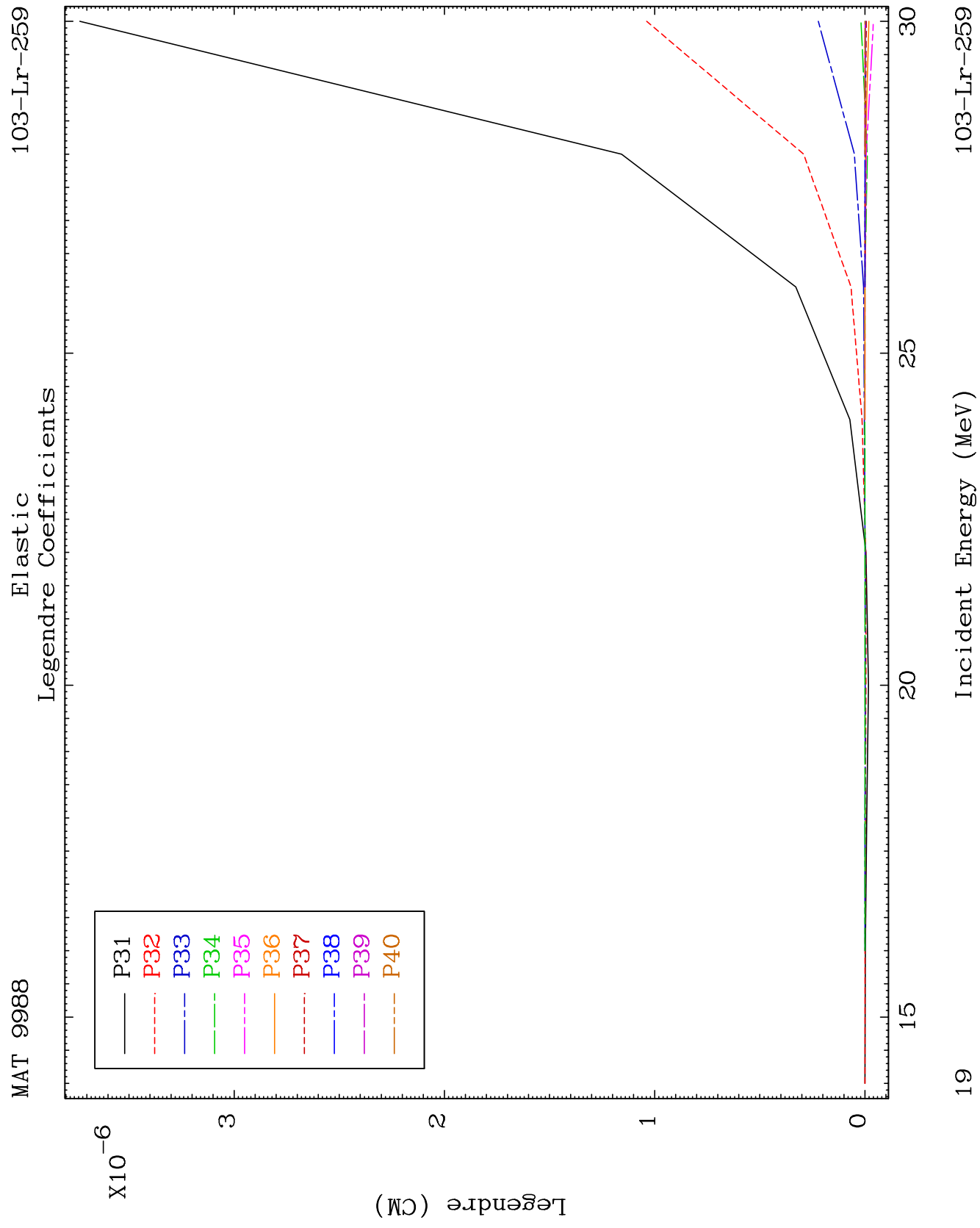


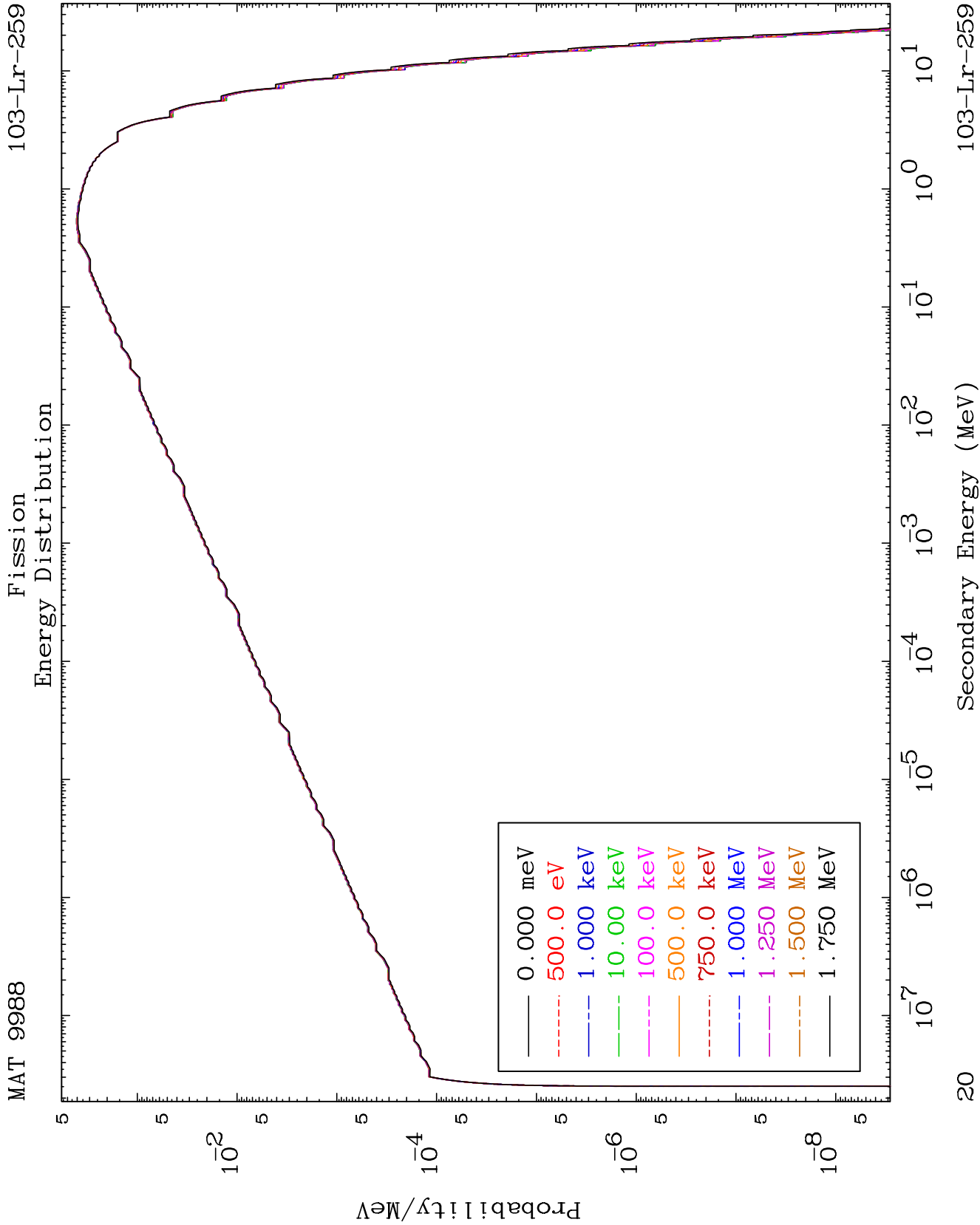


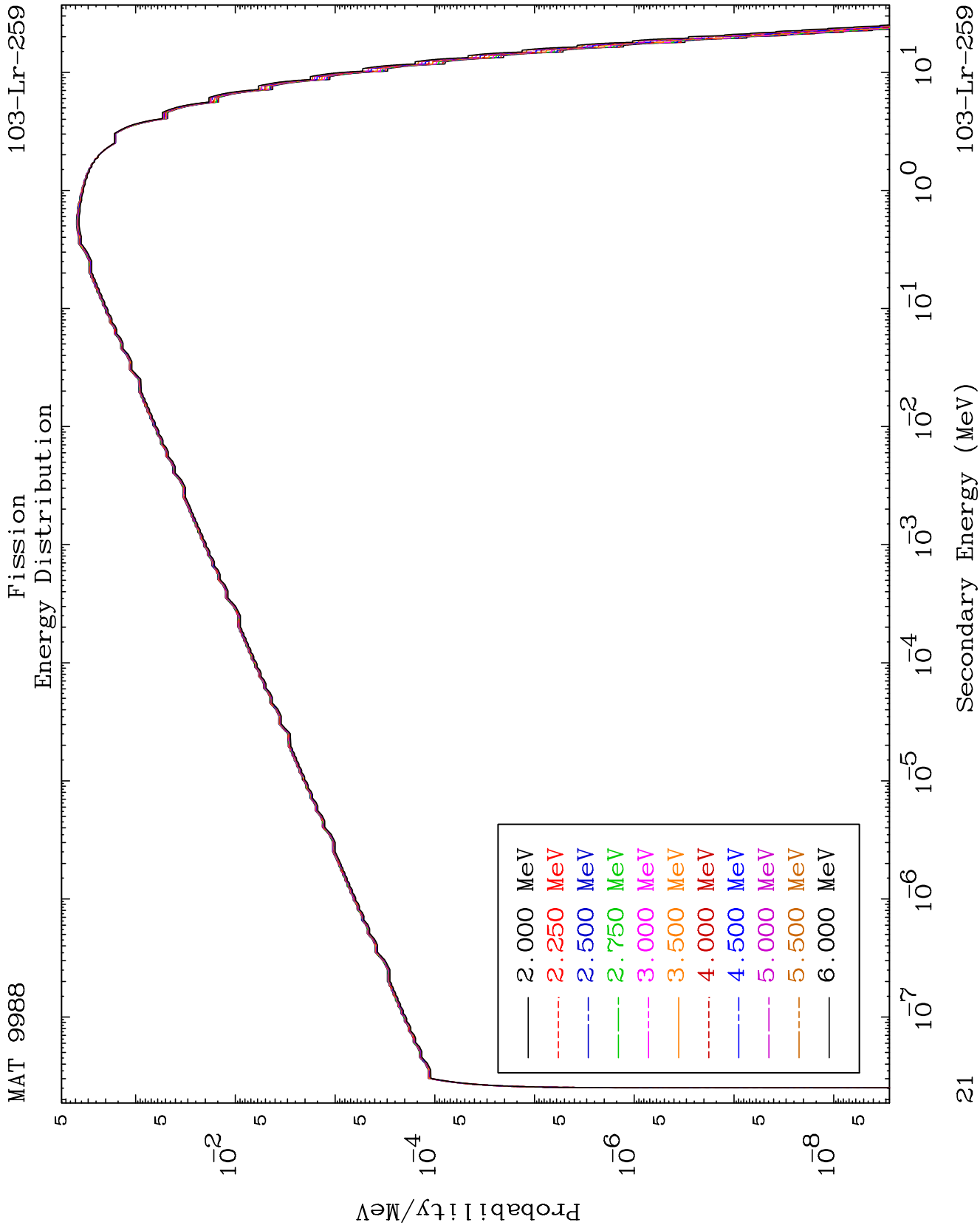


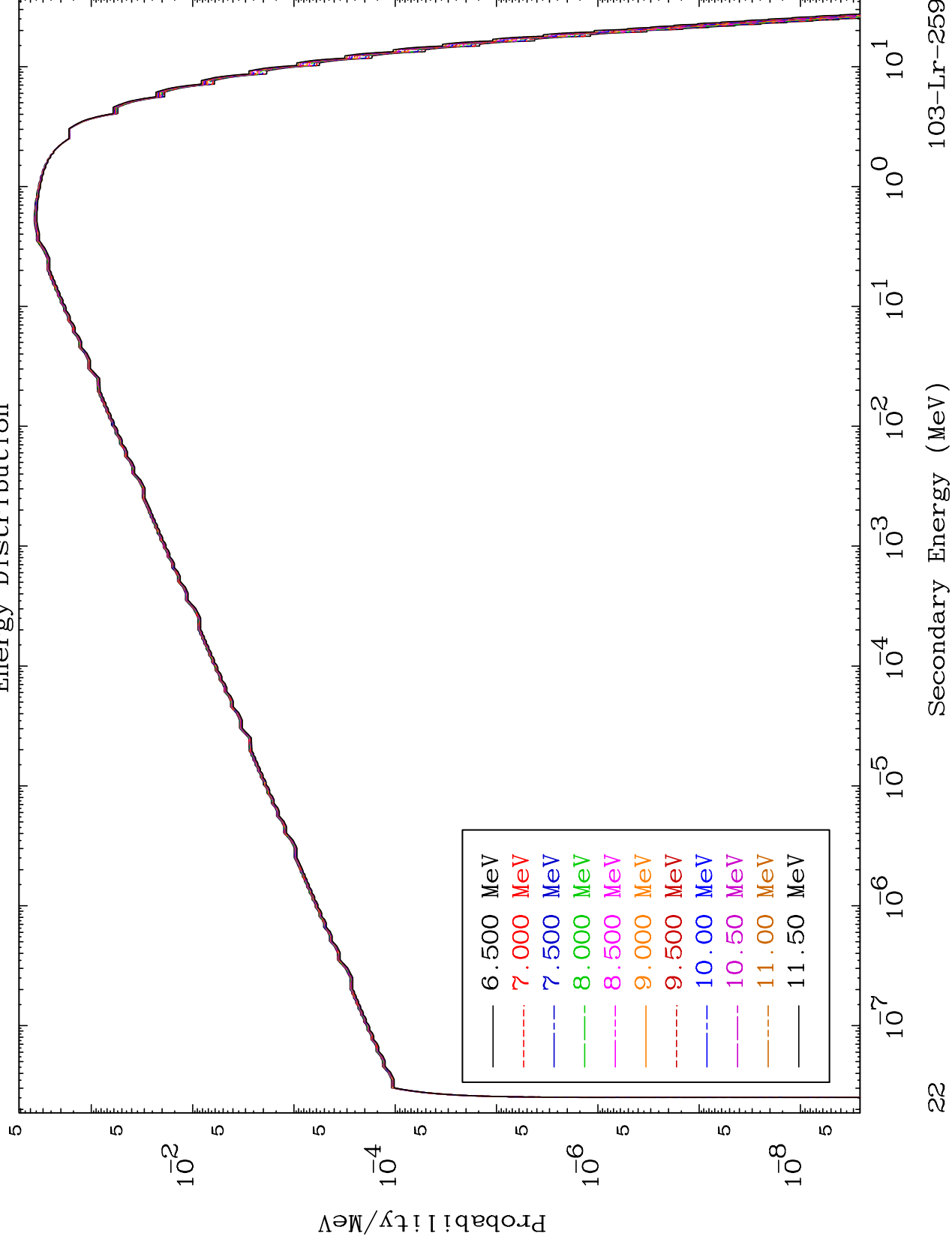








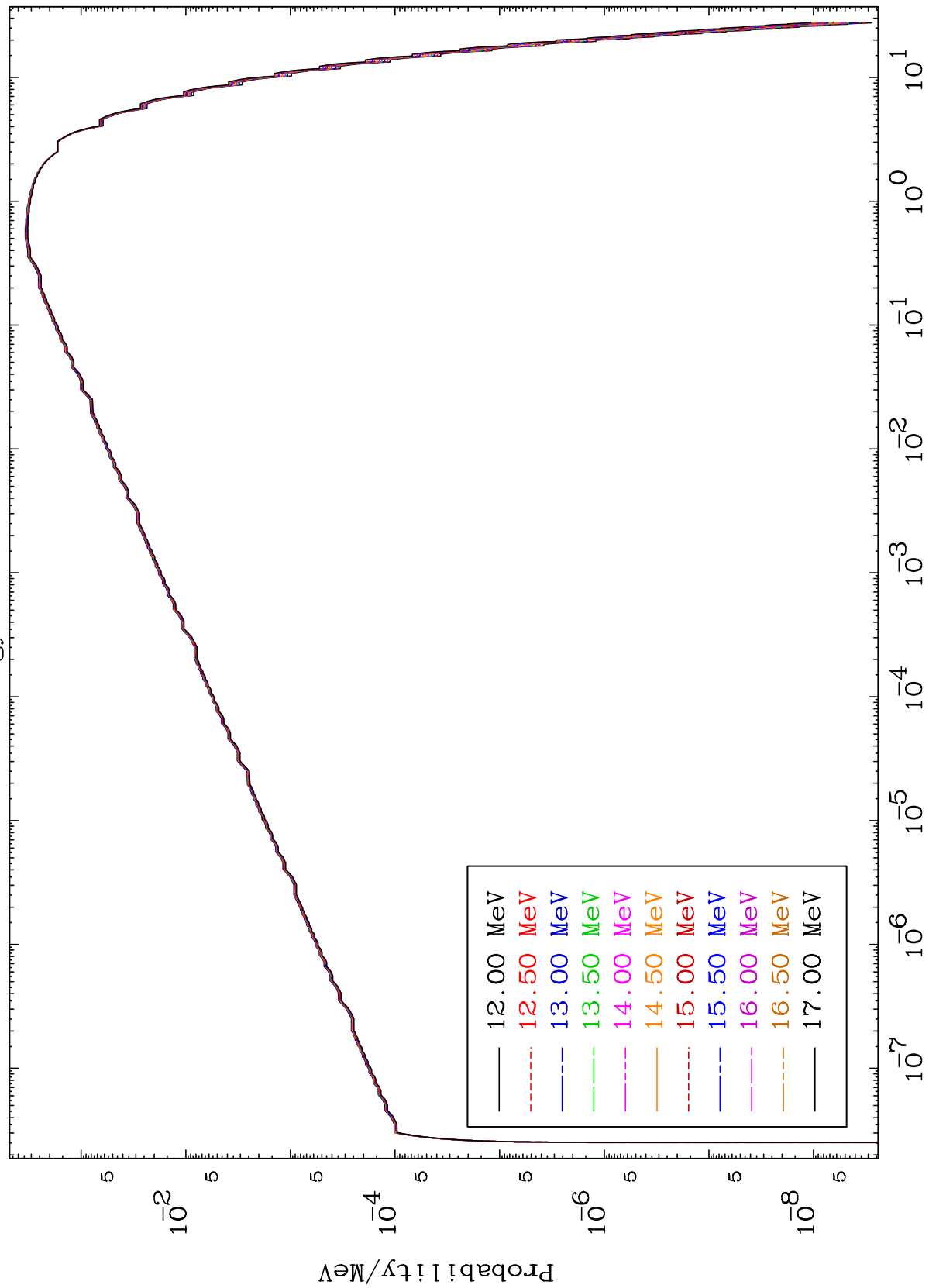




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Fission
Energy Distribution

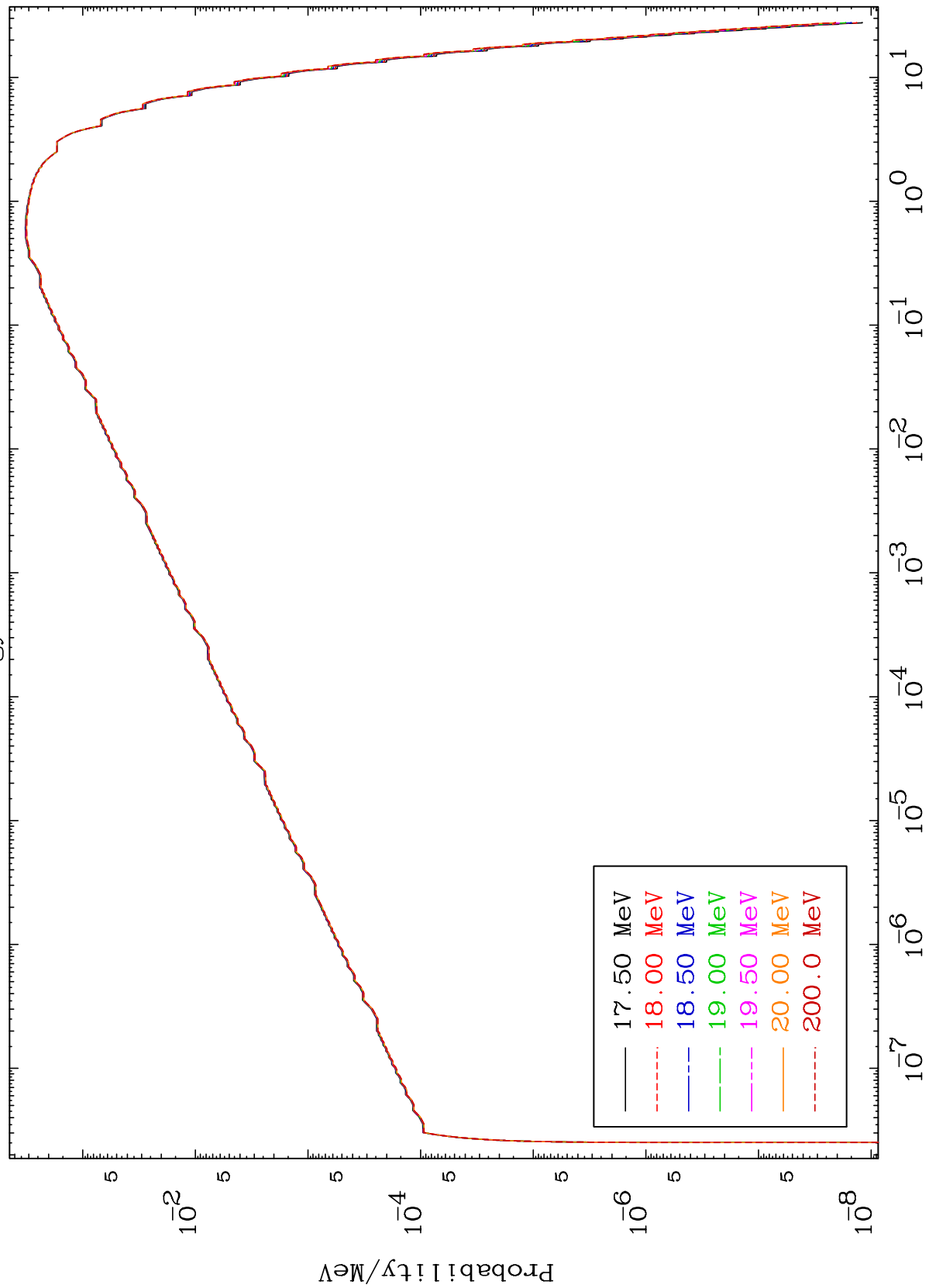
103-Lr-259



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Fission
Energy Distribution

103-Lr-259

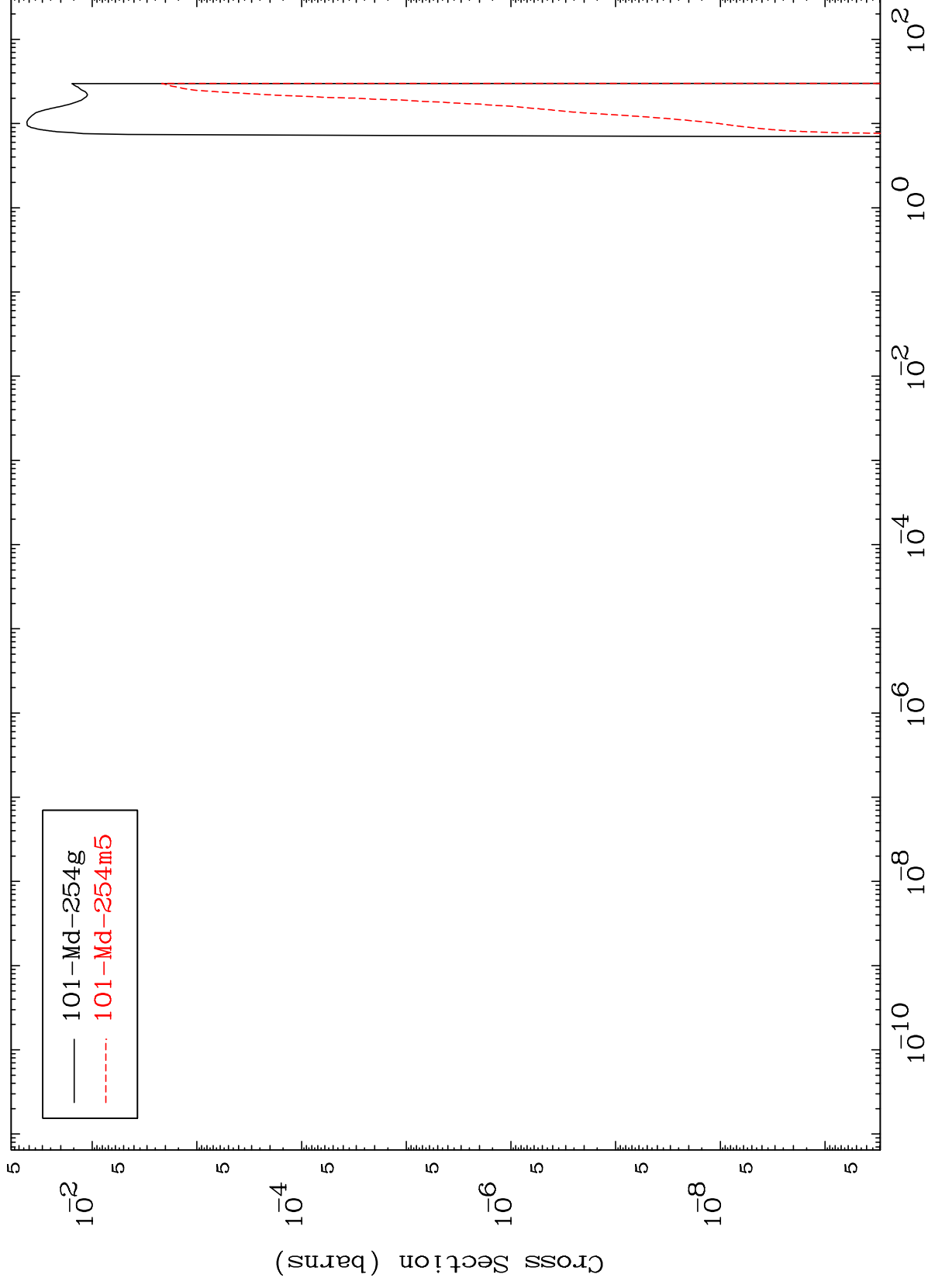


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$$(n, 2n) \propto$$

103-Lr-259

Radionuclide Production Cross Section



25

Incident Energy (MeV)

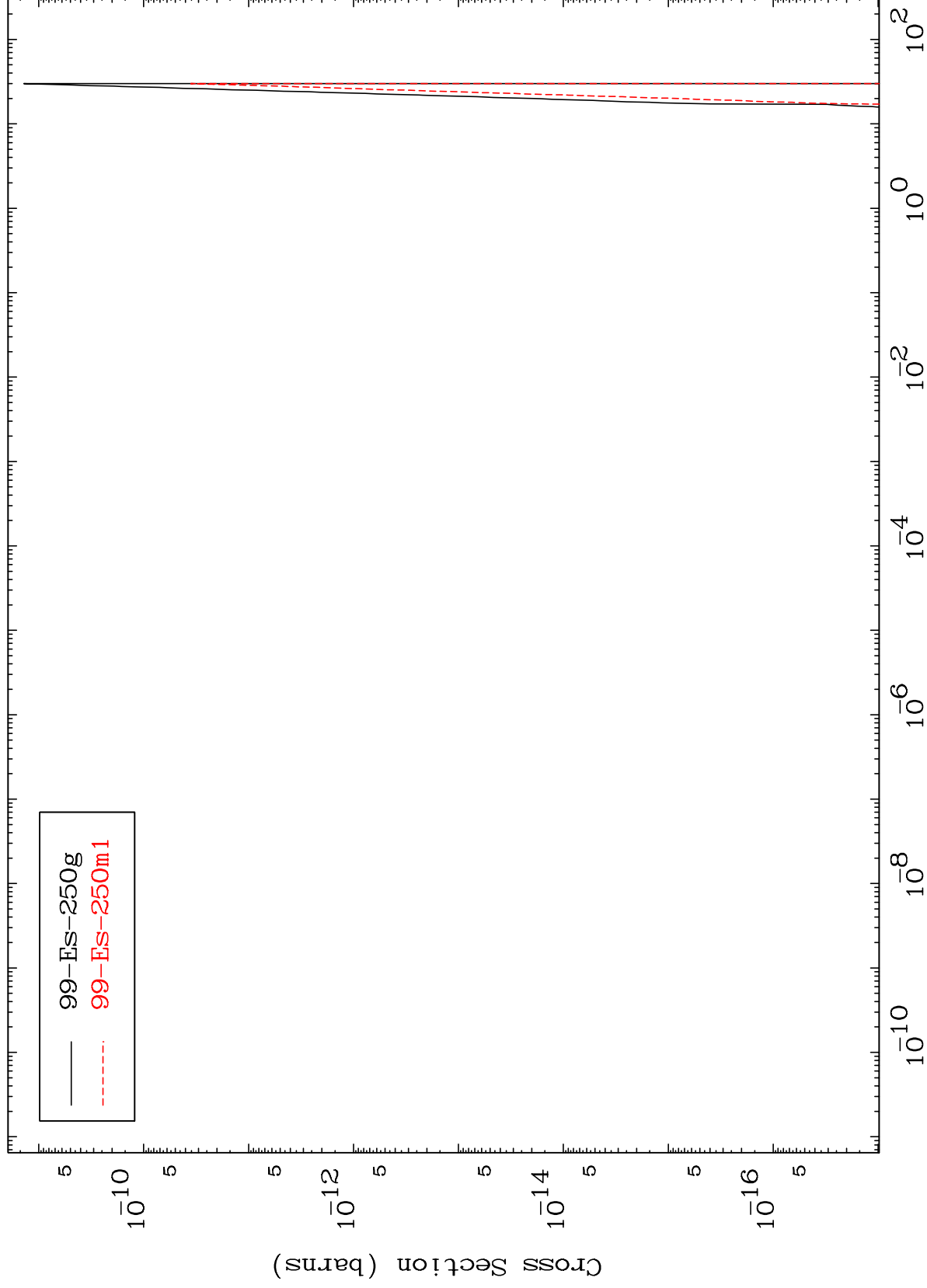
103-Lr-259

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(n,2n) 2 α

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Radionuclide Production Cross Section



26

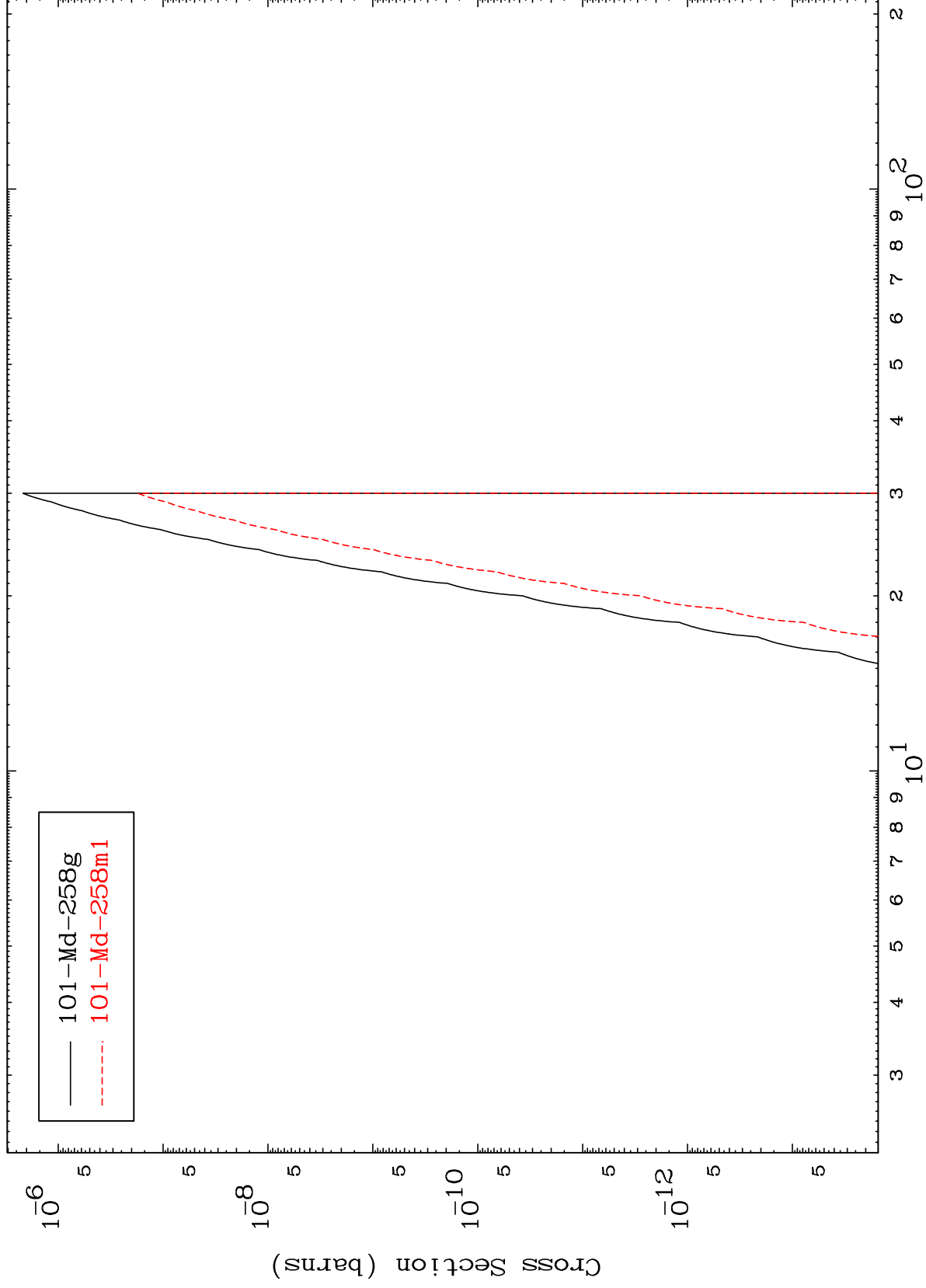
Incident Energy (MeV)

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103-Lr-259

(n,2p)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

103-Lr-259