

Program EVALPLOT  
(Version 2017-1)

by

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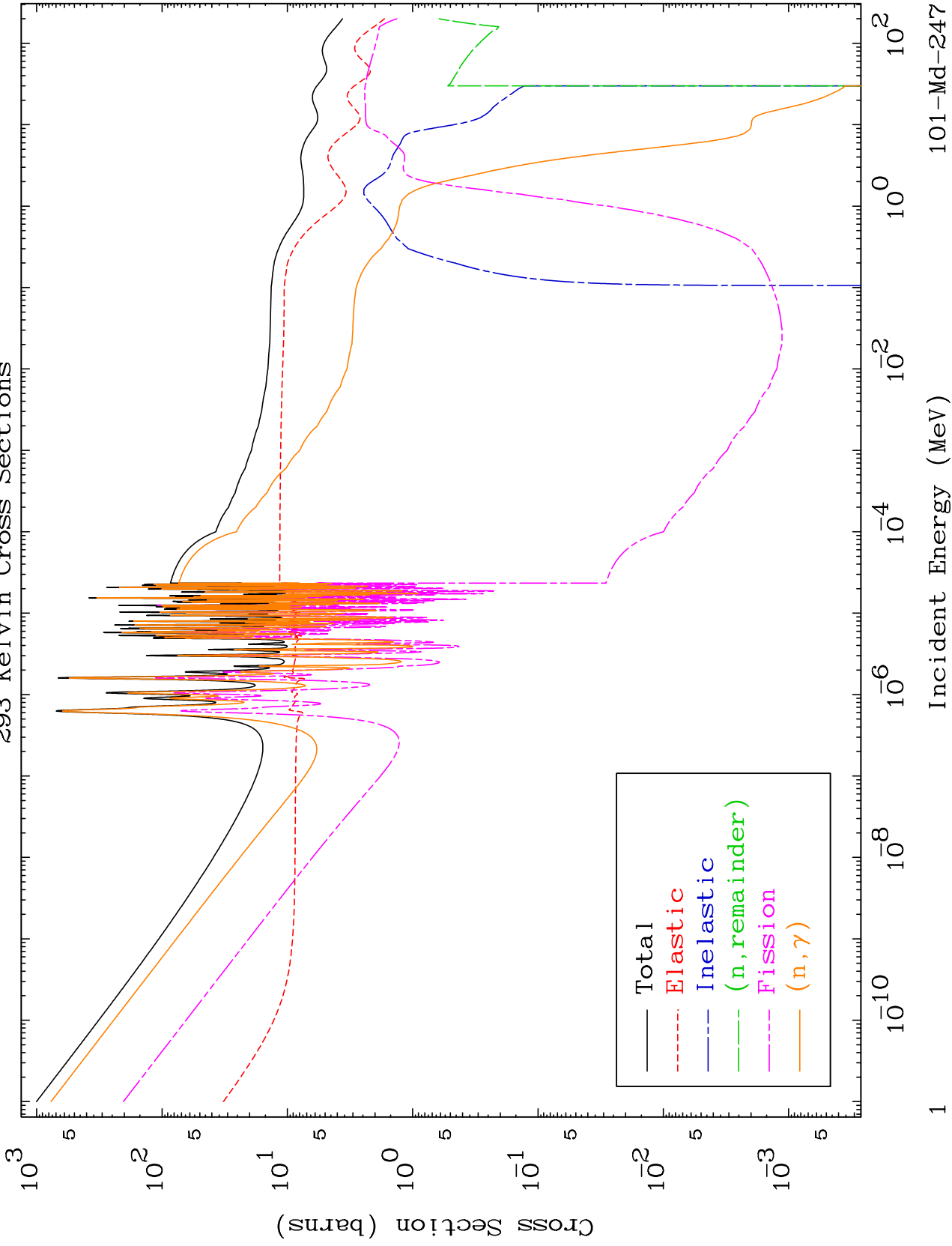
Press Mouse Button to Start

MAT 9948

Major

293 Kelvin Cross Sections

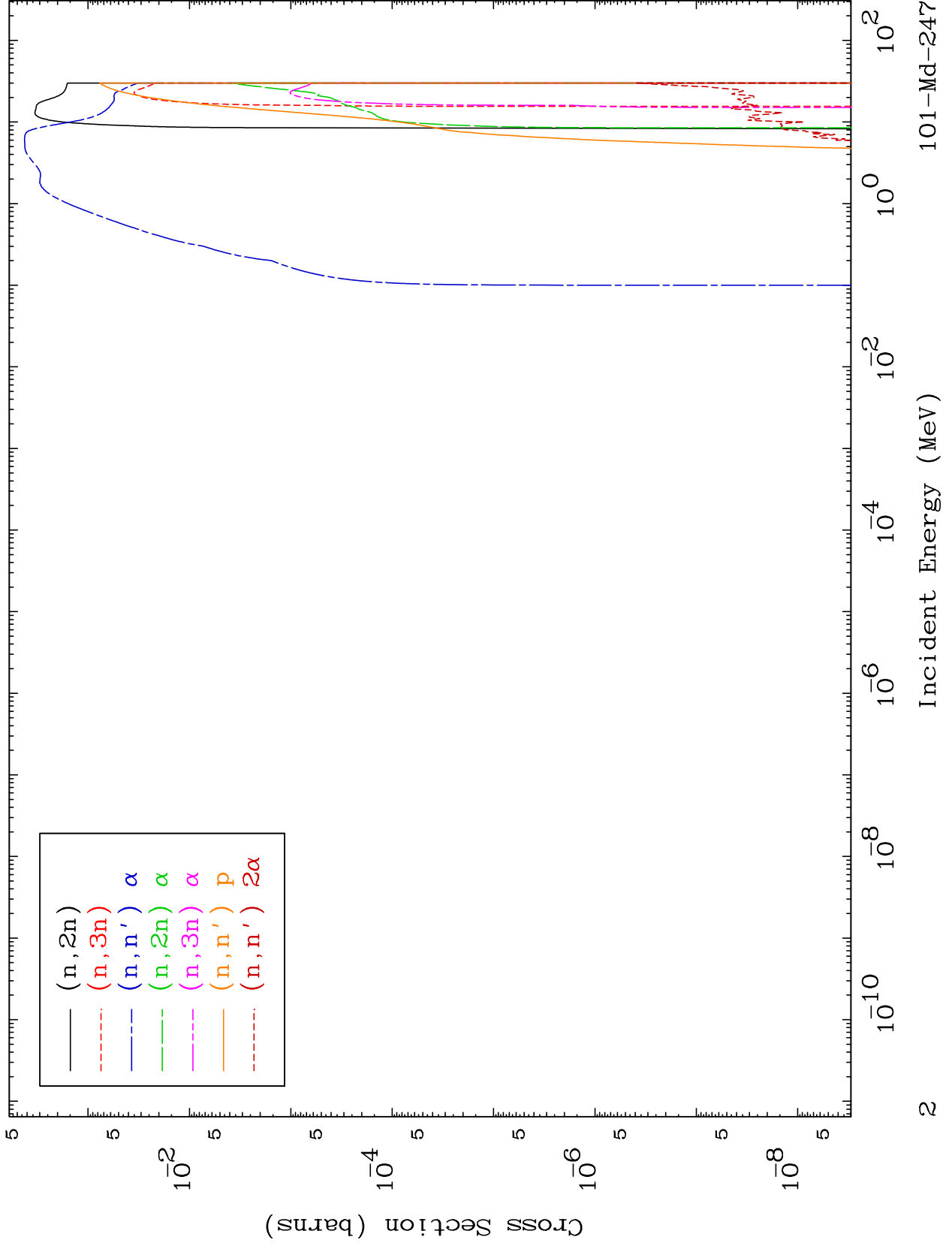
101-Md-247



MAT 9948

Neutron Production  
293 Kelvin Cross Sections

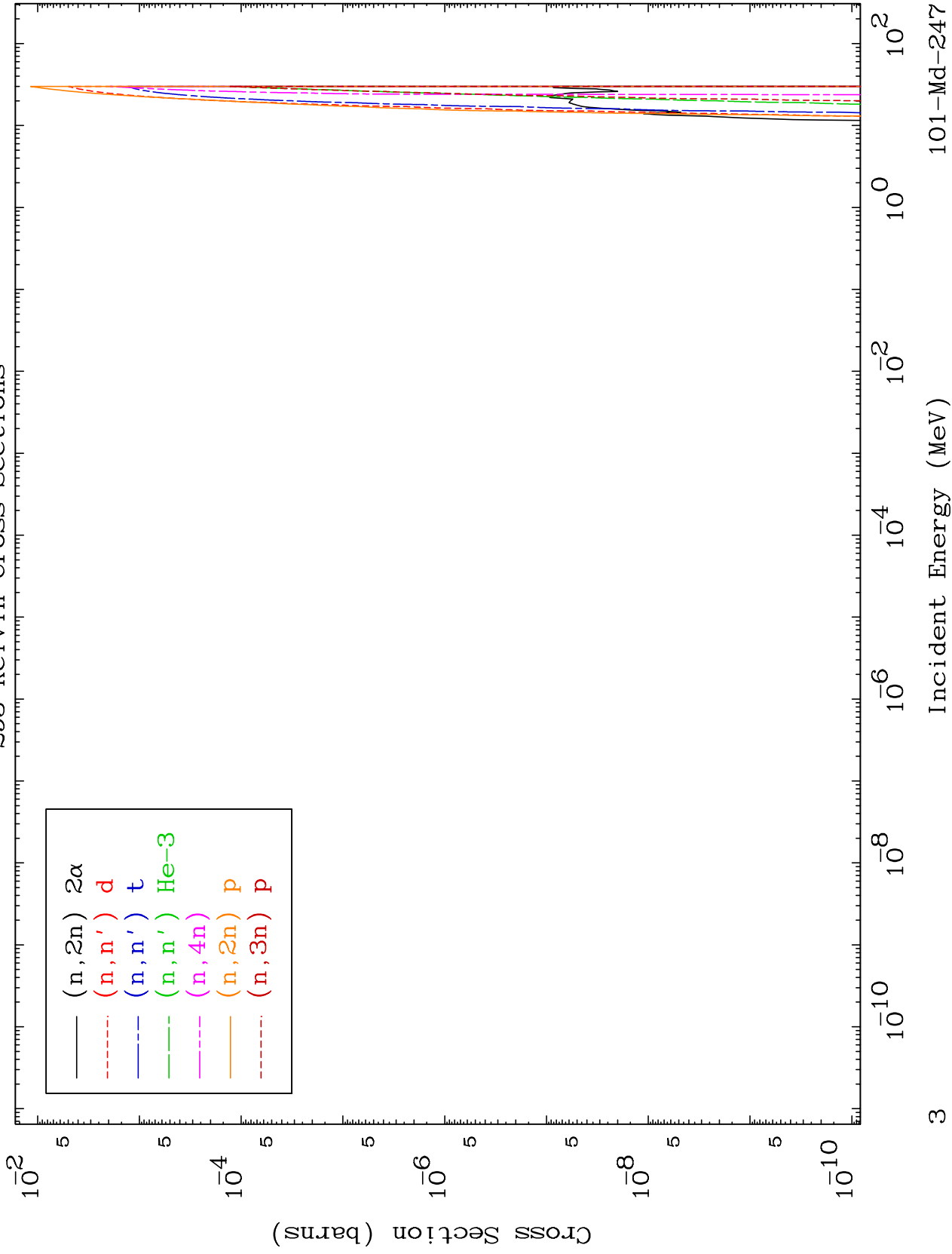
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MAT 9948

Neutron Production  
293 Kelvin Cross Sections

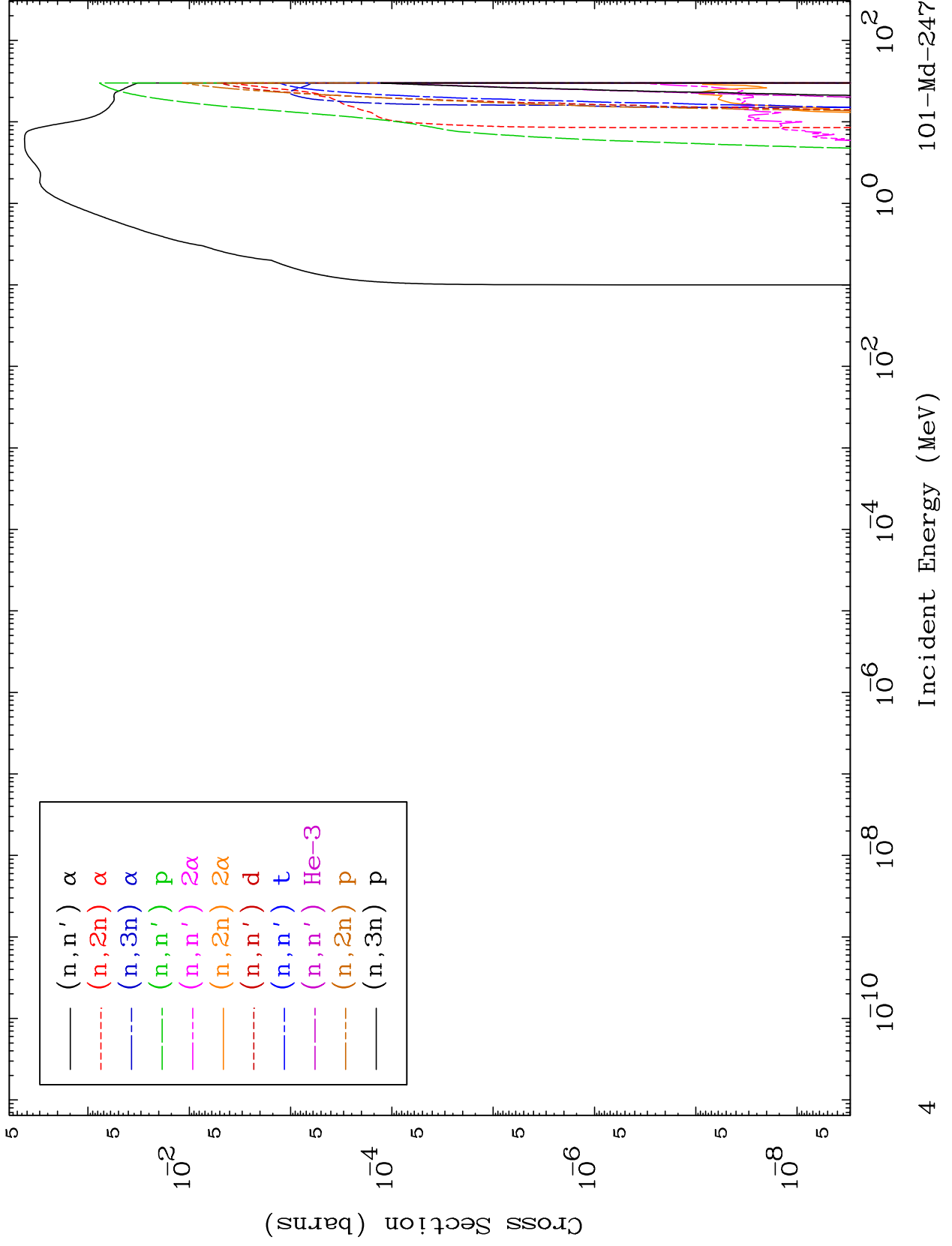
101-Md-247



MAT 9948

Charged Particle  
293 Kelvin Cross Sections

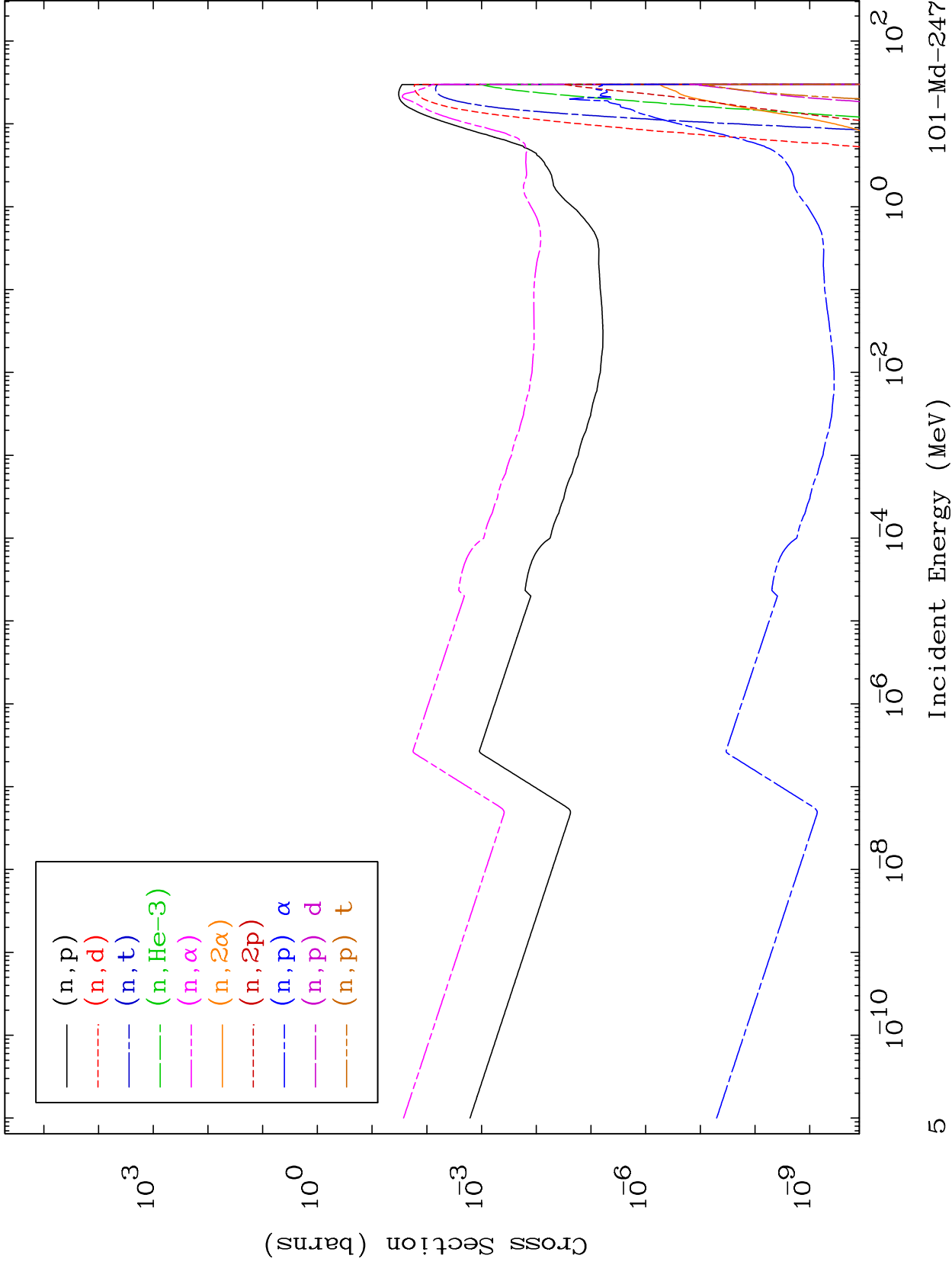
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Charged Particle  
293 Kelvin Cross Sections

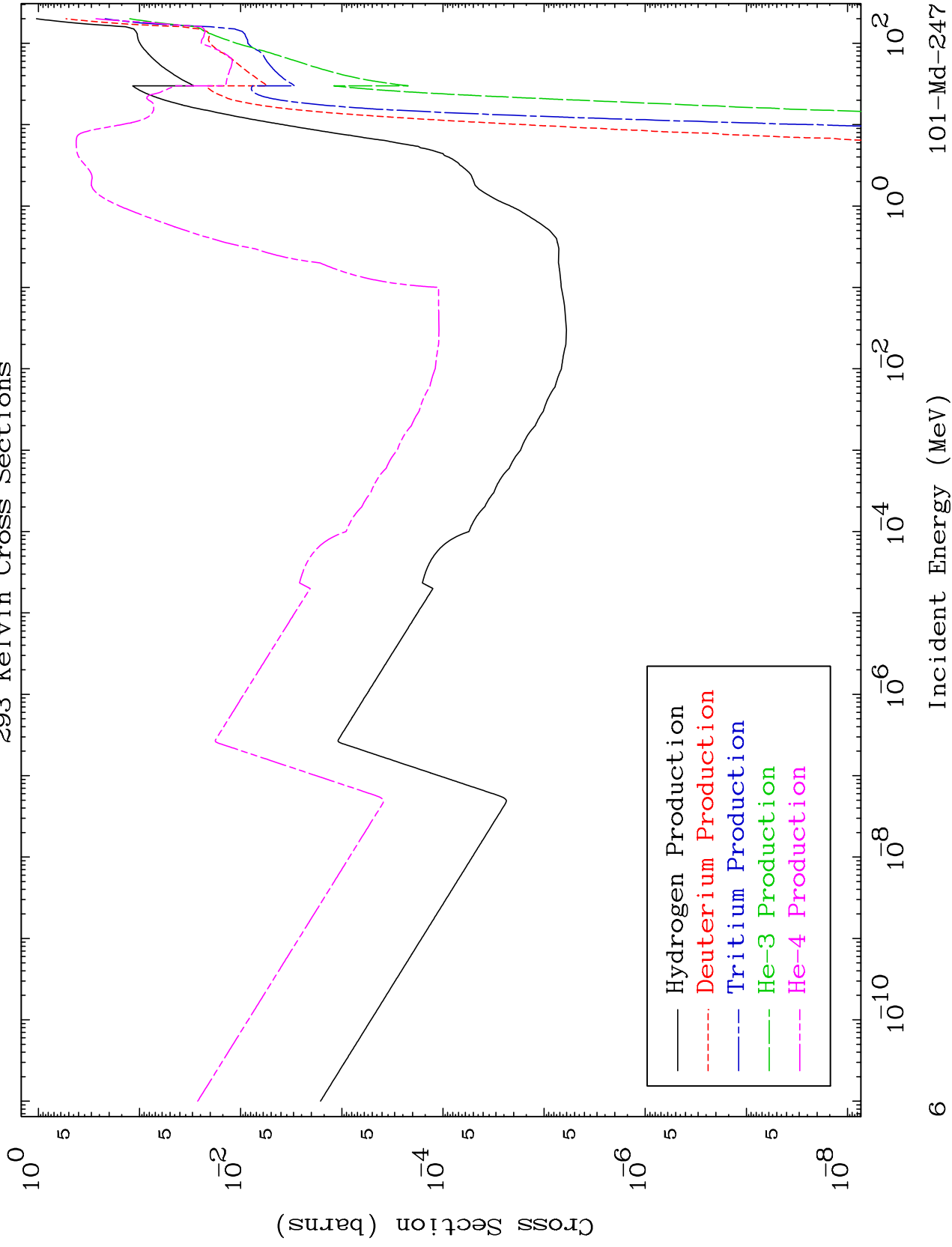
101-Md-247



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Particle Production  
293 Kelvin Cross Sections

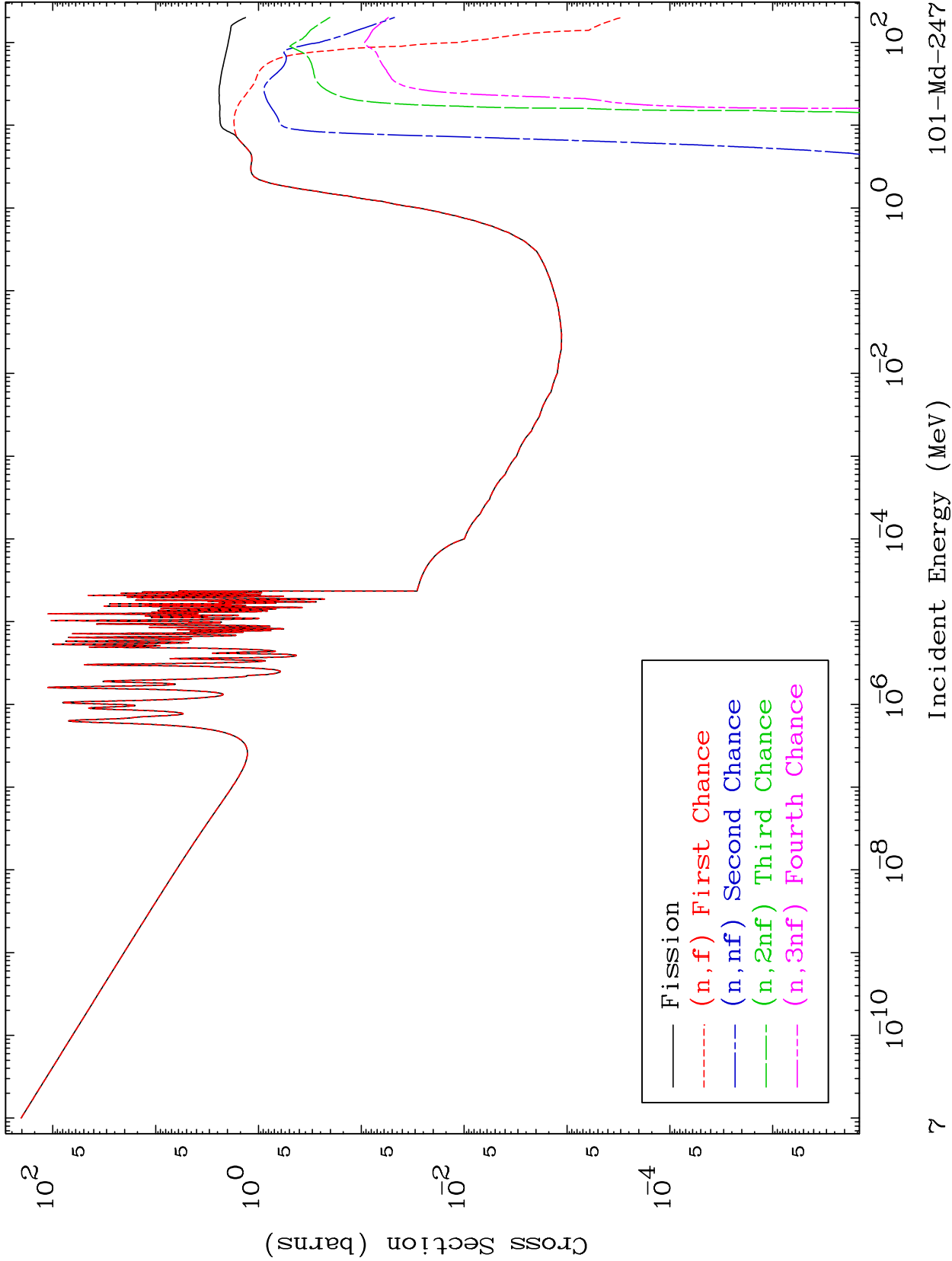
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Fission  
293 Kelvin Cross Sections

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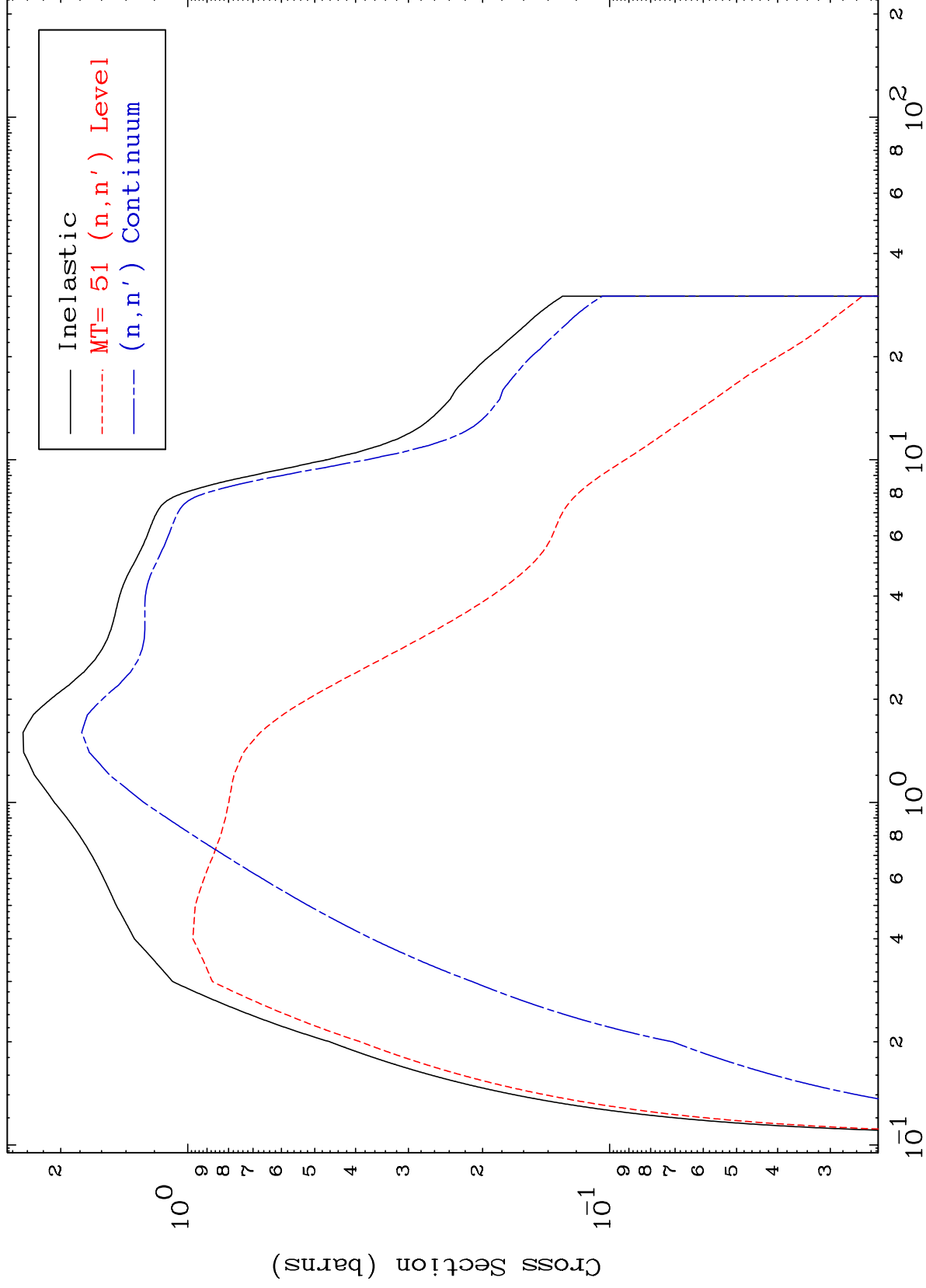


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(n,n') Level

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293 Kelvin Cross Sections



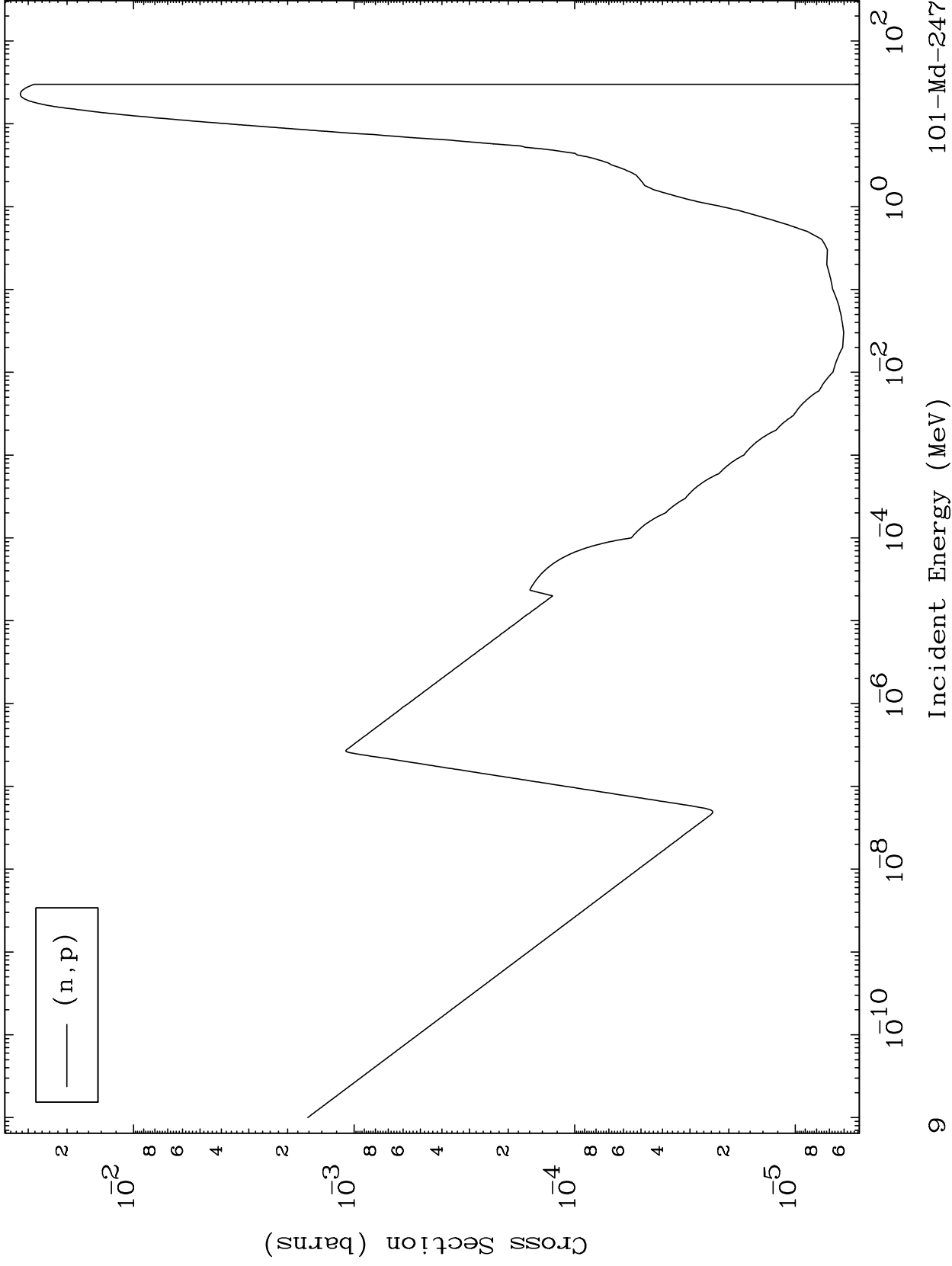
Incident Energy (MeV)

101-Md-247

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(n,p) Levels  
293 Kelvin Cross Sections

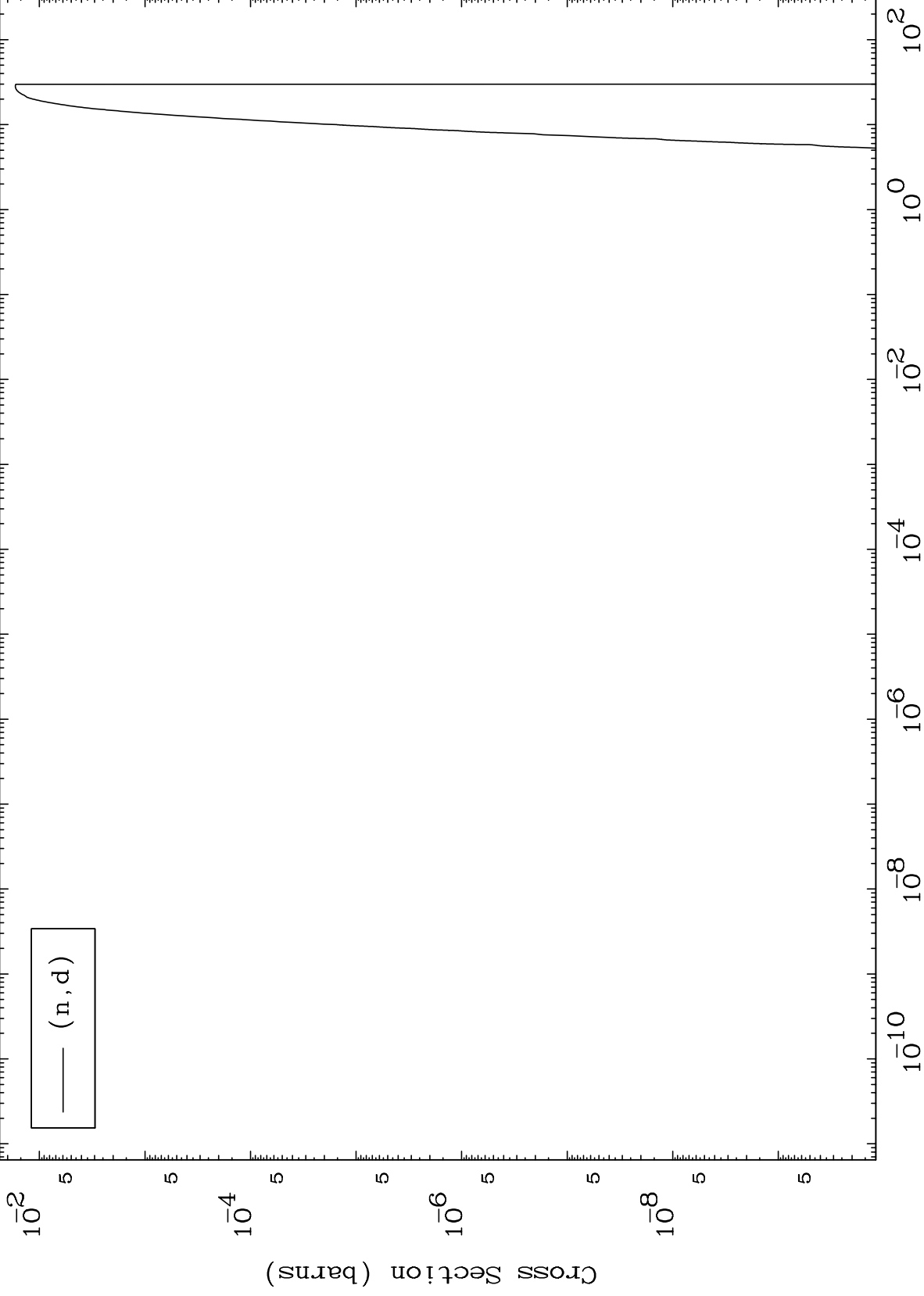
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(n,d) Levels  
293 Kelvin Cross Sections

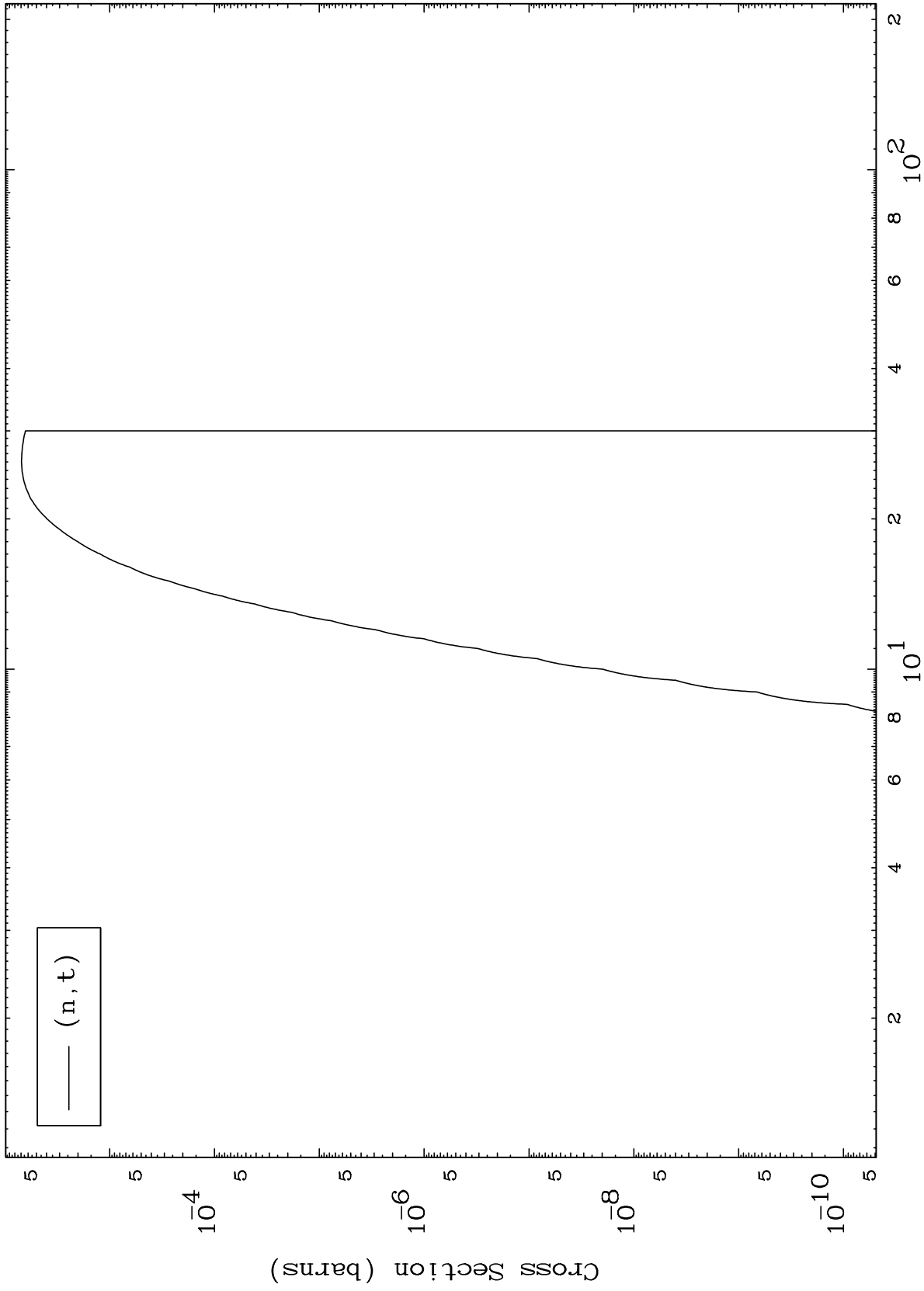
101-Md-247



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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-247



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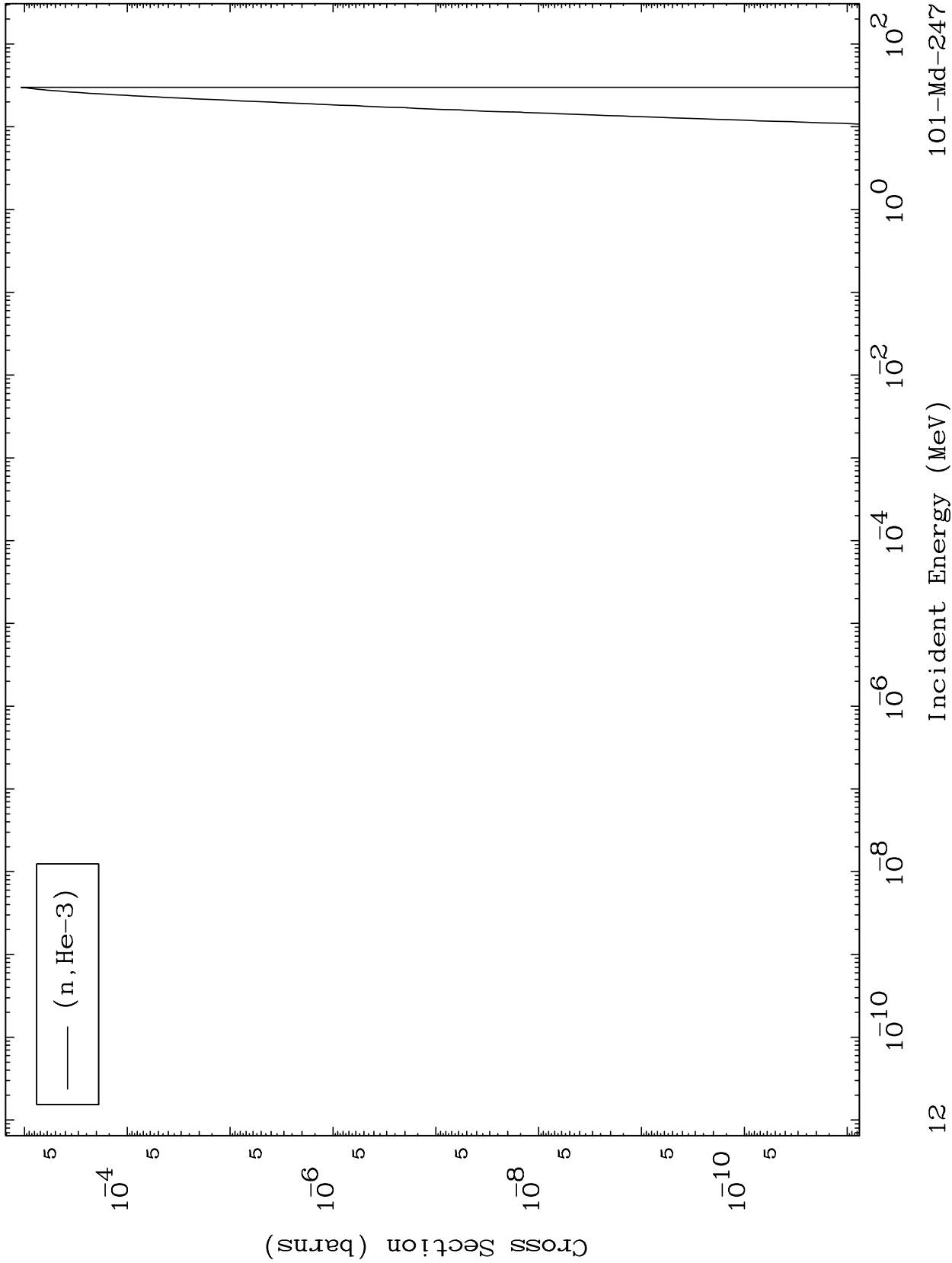
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,He3) Levels  
293 Kelvin Cross Sections

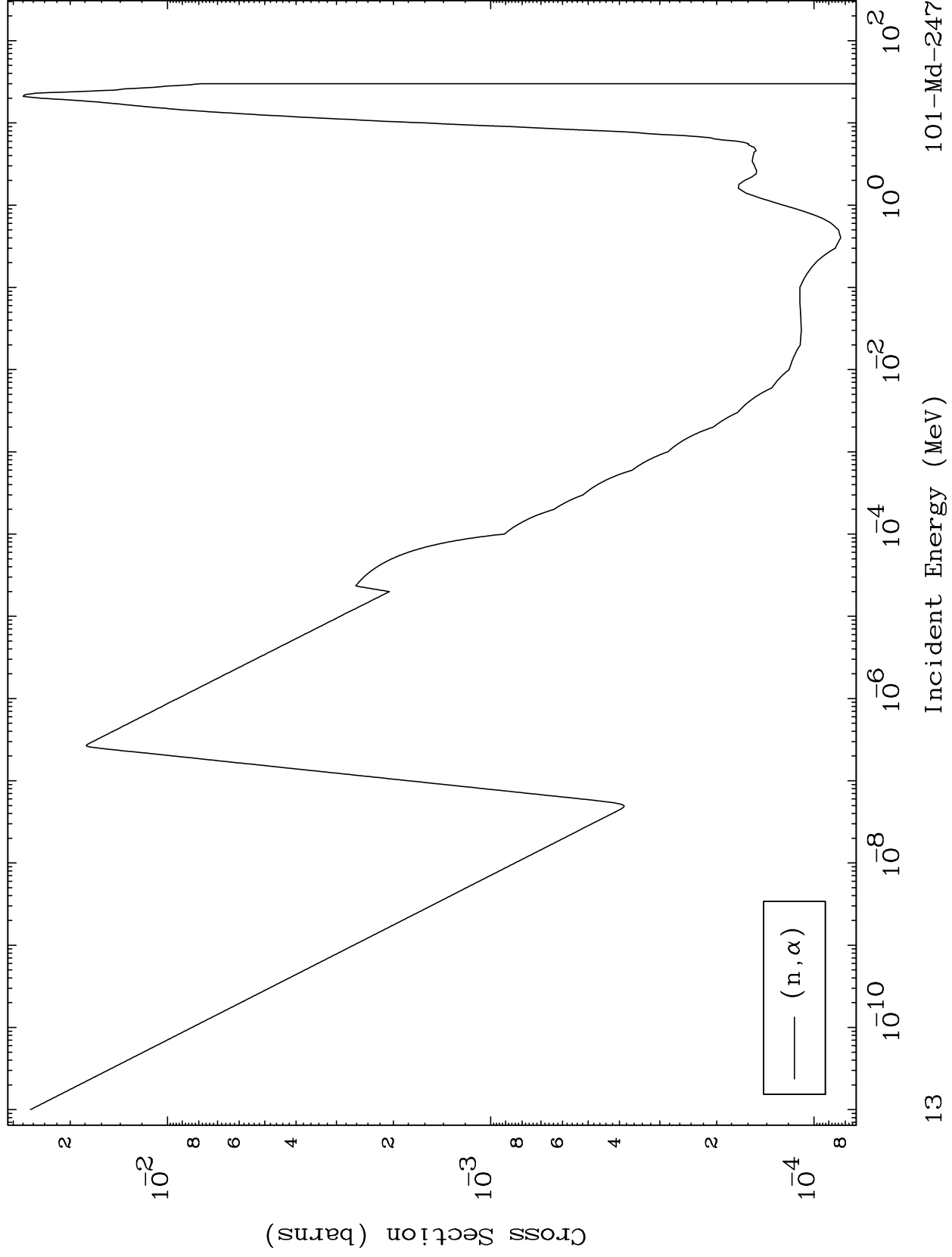
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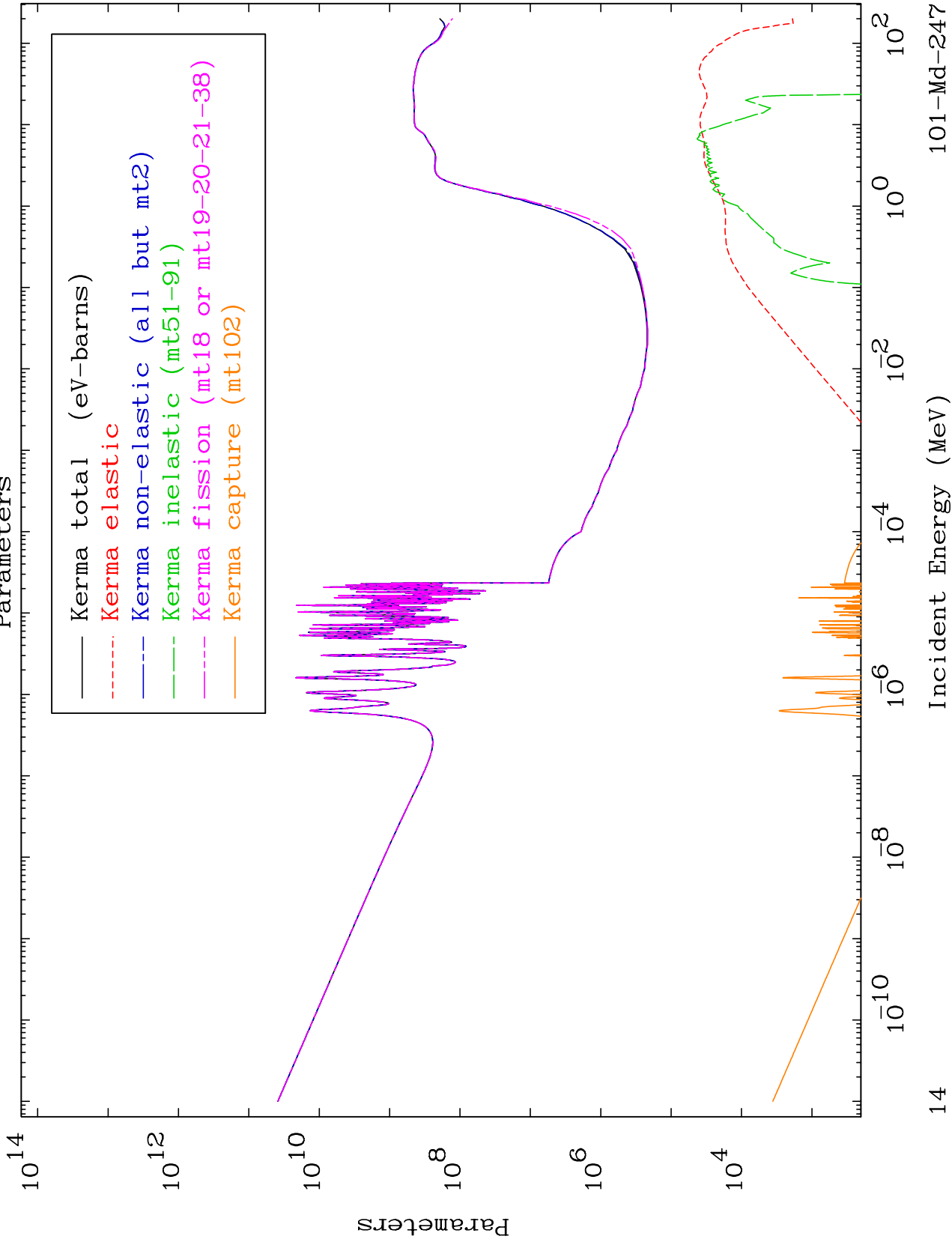
MAT 9948

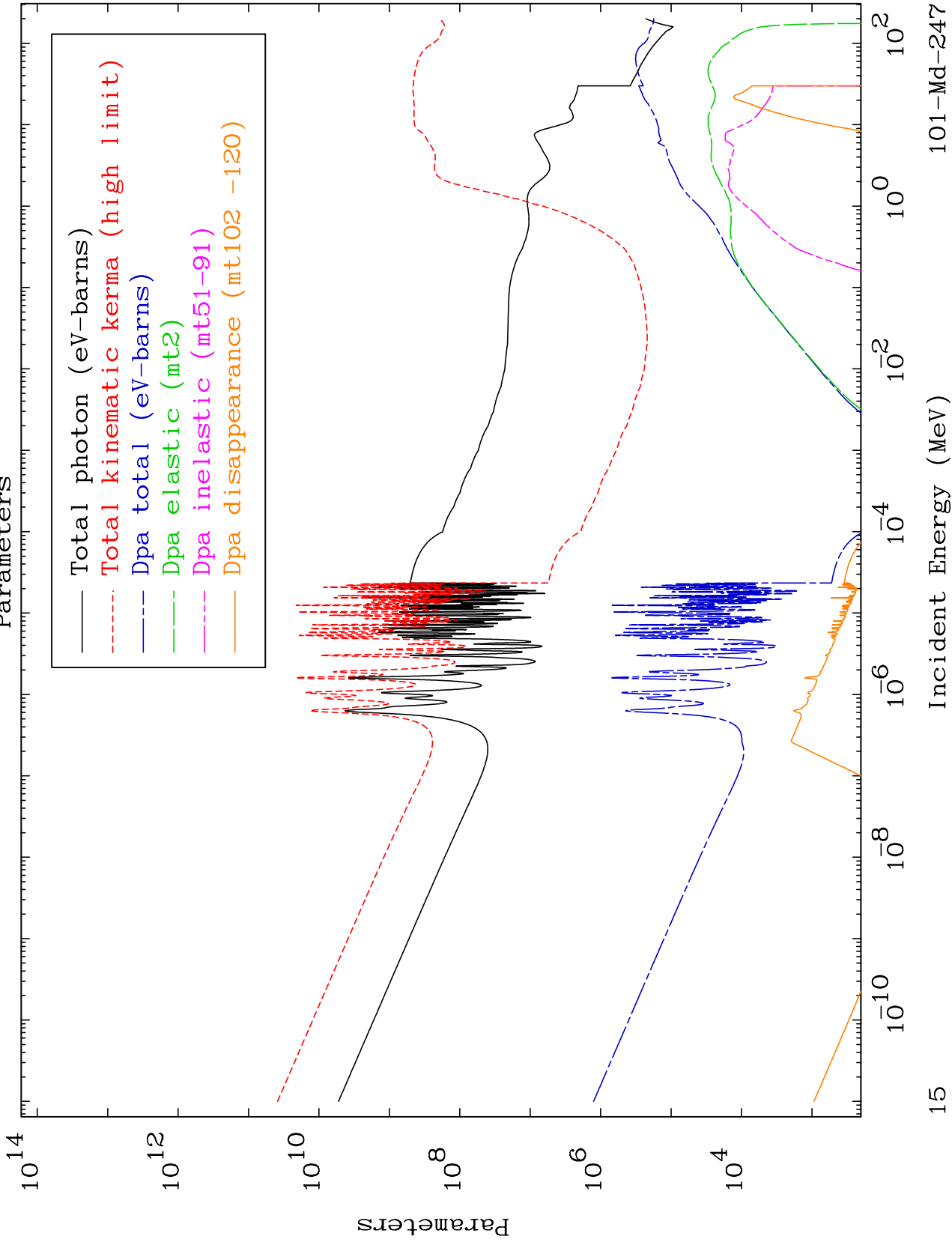
(n,α) Levels  
293 Kelvin Cross Sections

101-Md-247



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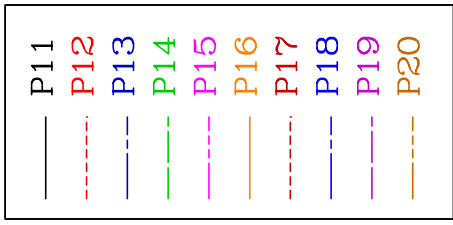
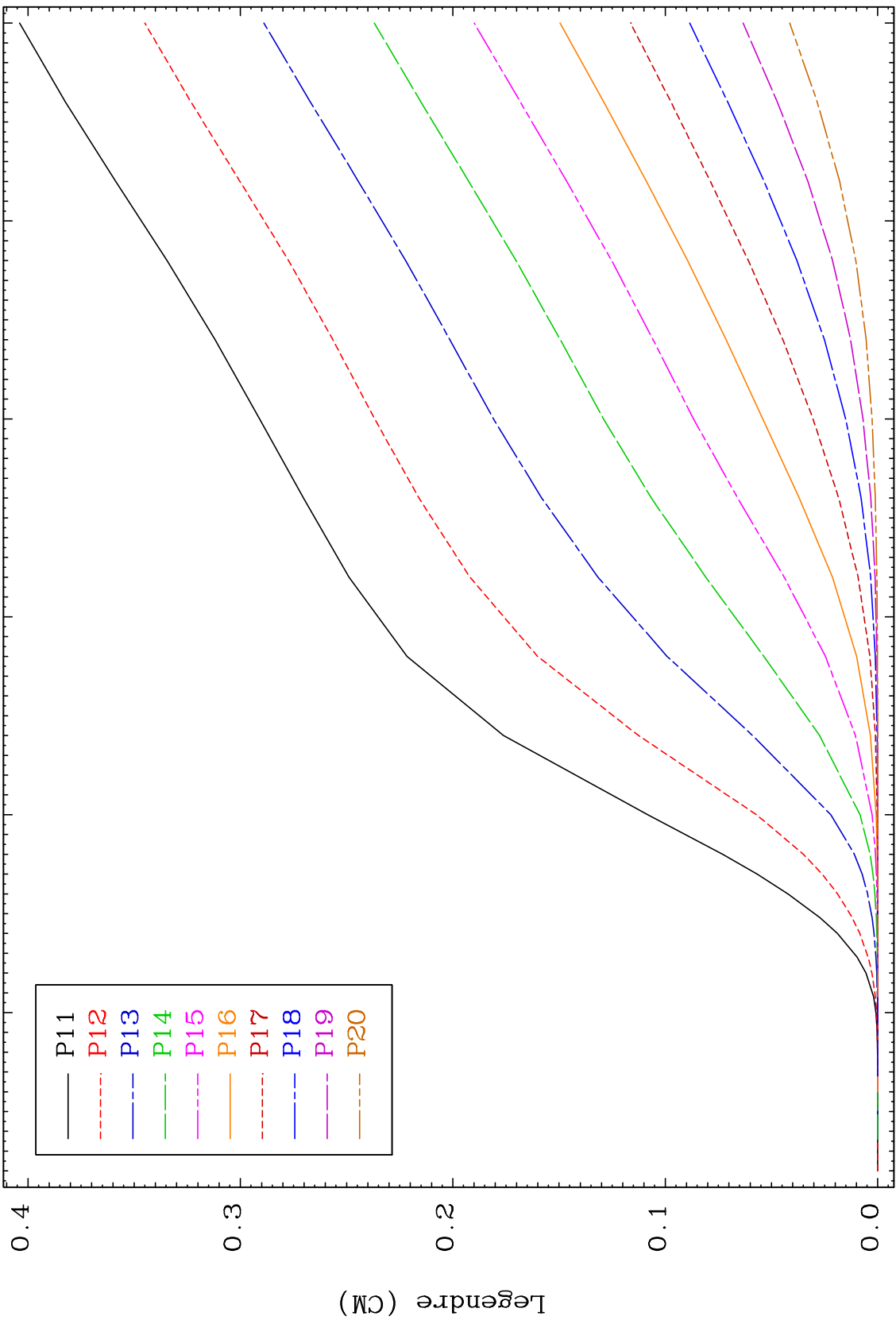




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Elastic Legendre Coefficients

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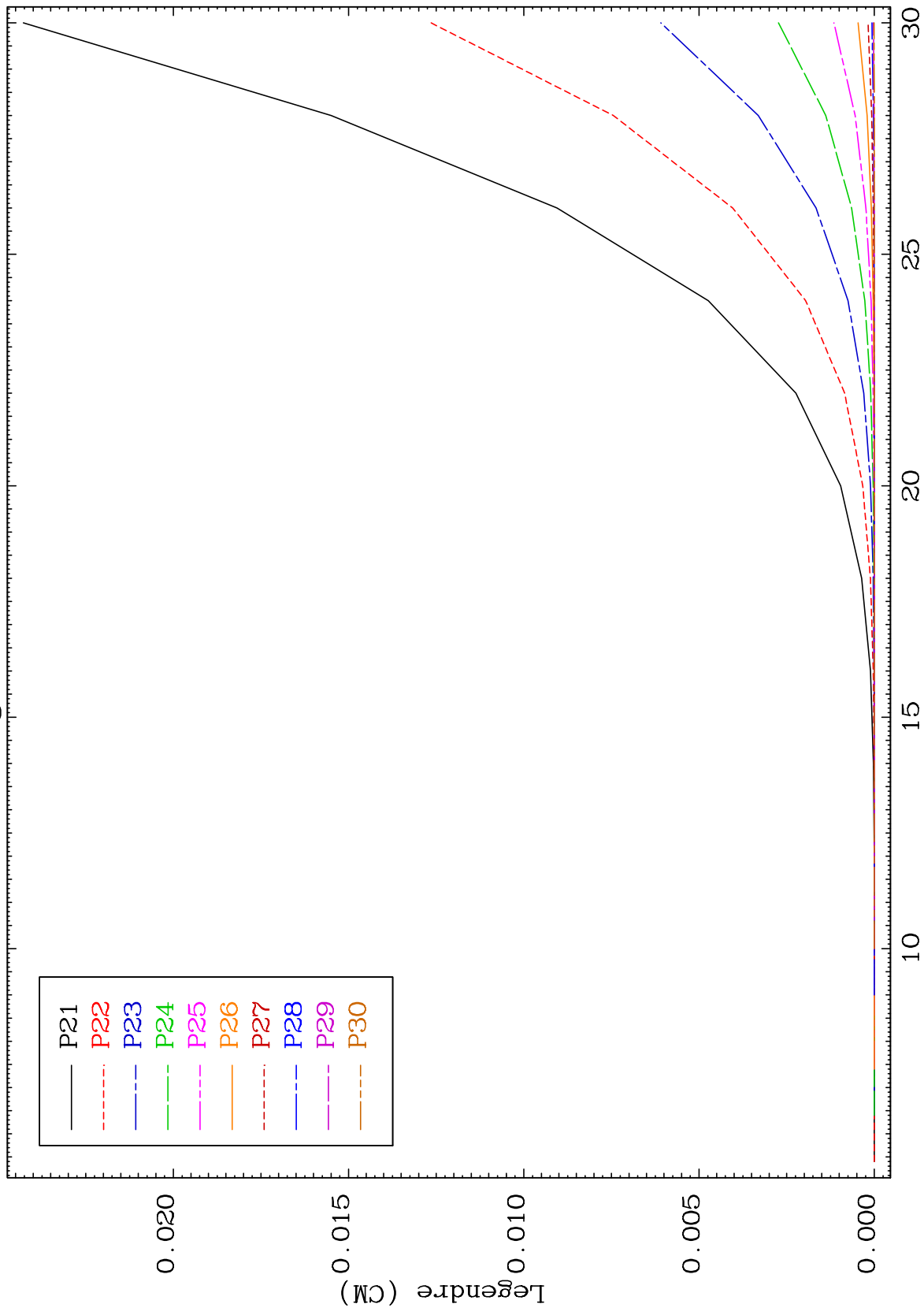
Incident Energy (MeV)

101-Md-247

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Elastic Legendre Coefficients

101-Md-247



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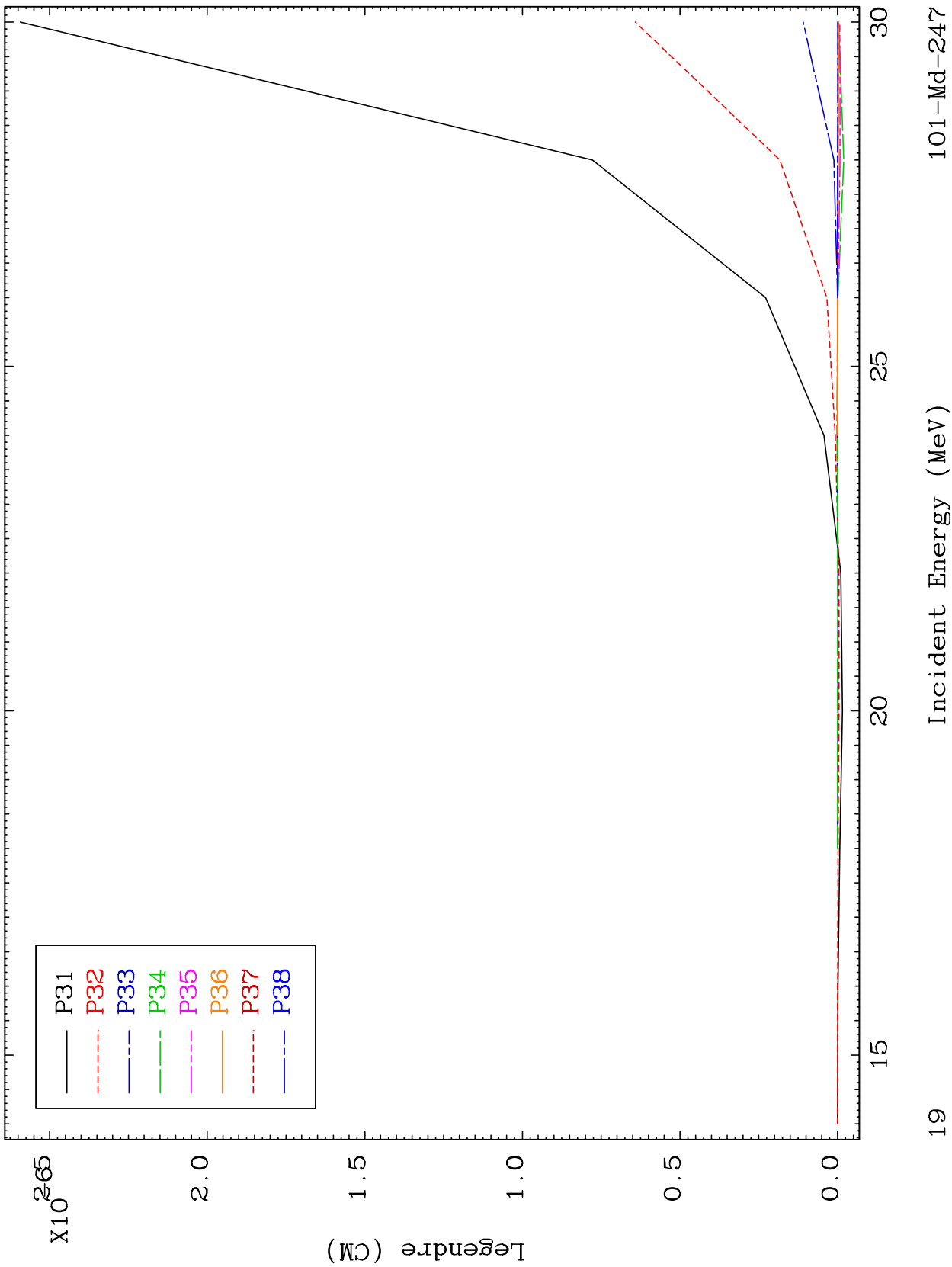
Incident Energy (MeV)

101-Md-247

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Elastic Legendre Coefficients

101-Md-247



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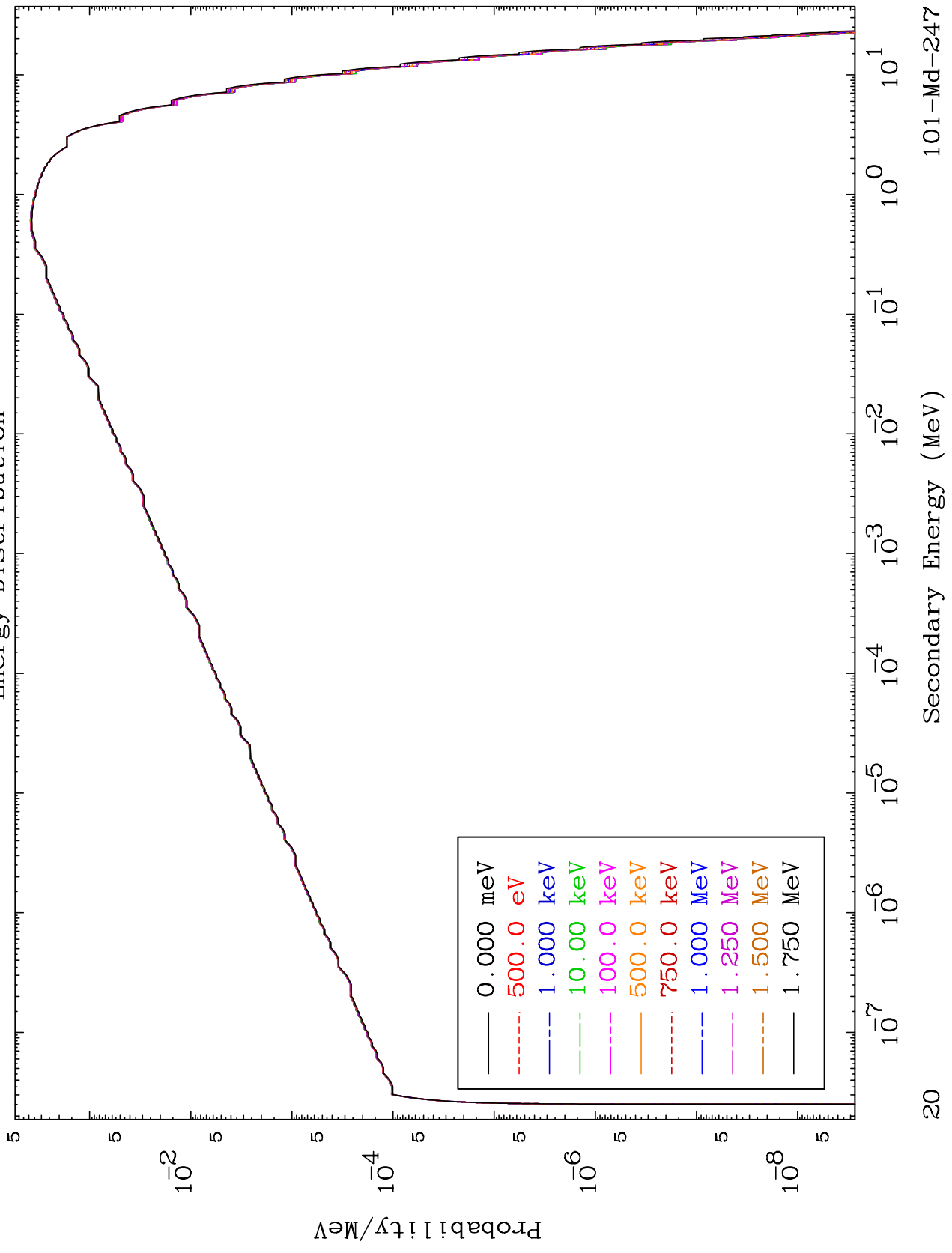
Incident Energy (MeV)

101-Md-247

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### Fission Energy Distribution

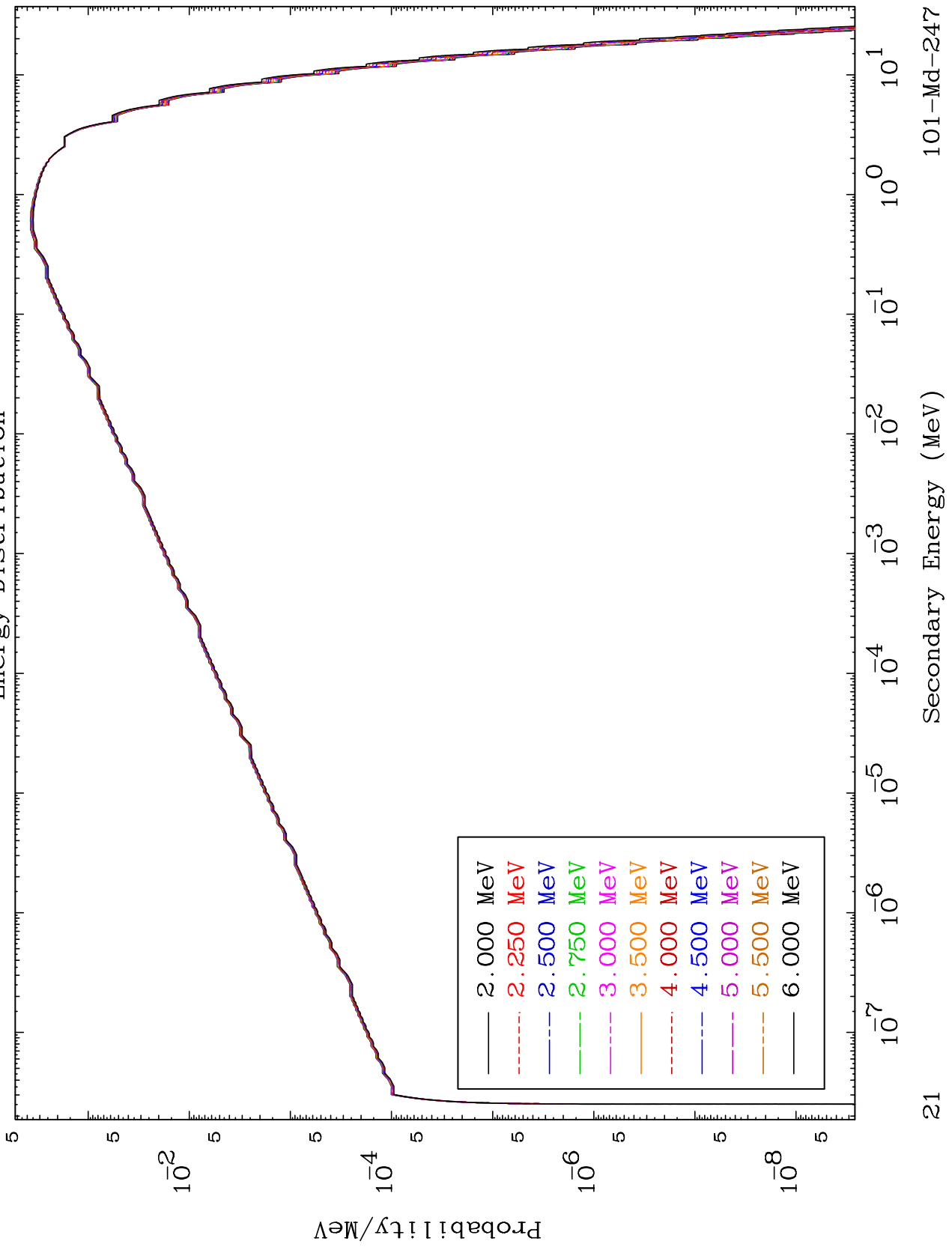
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### Fission Energy Distribution

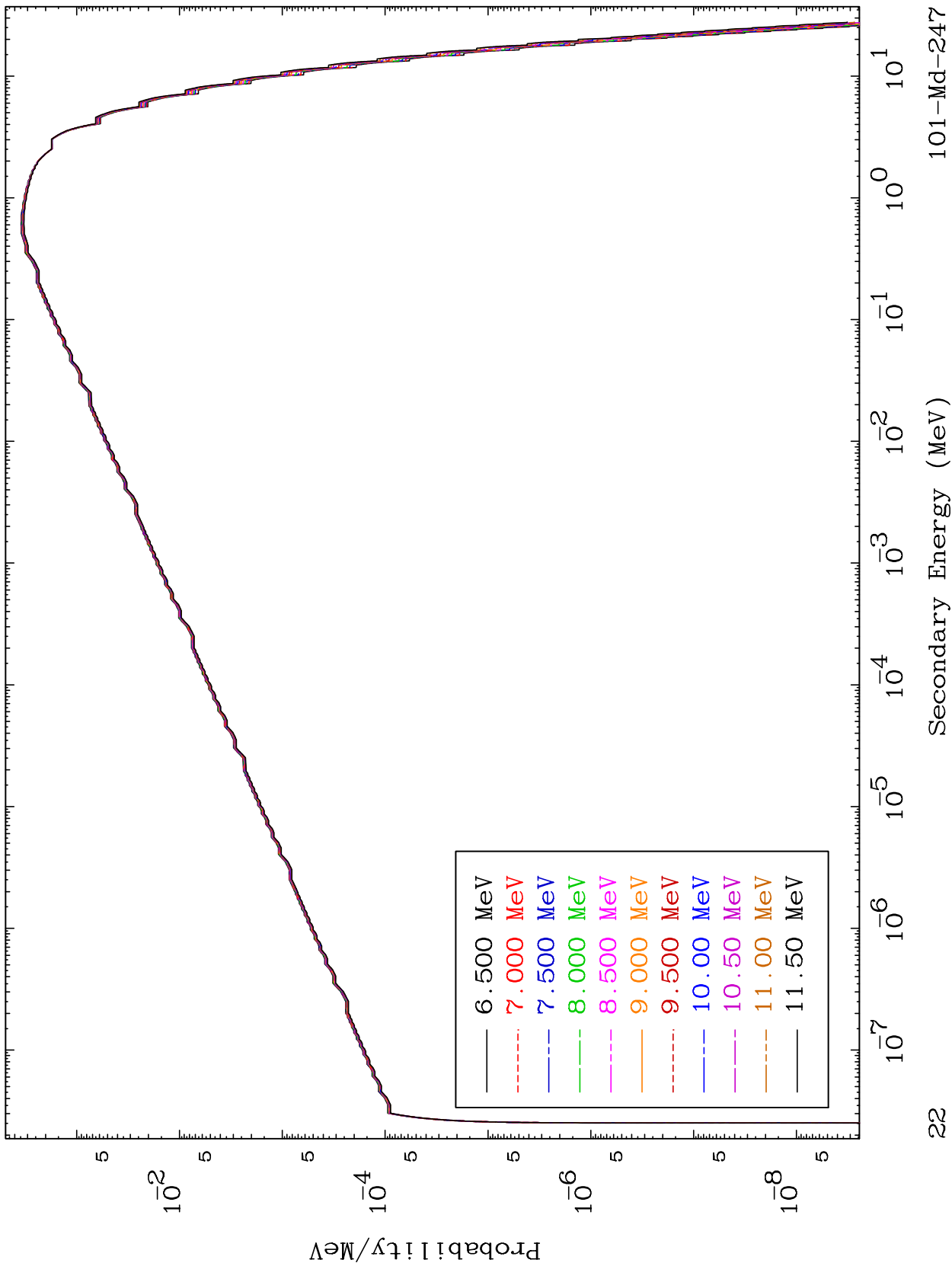
101-Md-247



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Fission  
Energy Distribution

101-Md-247



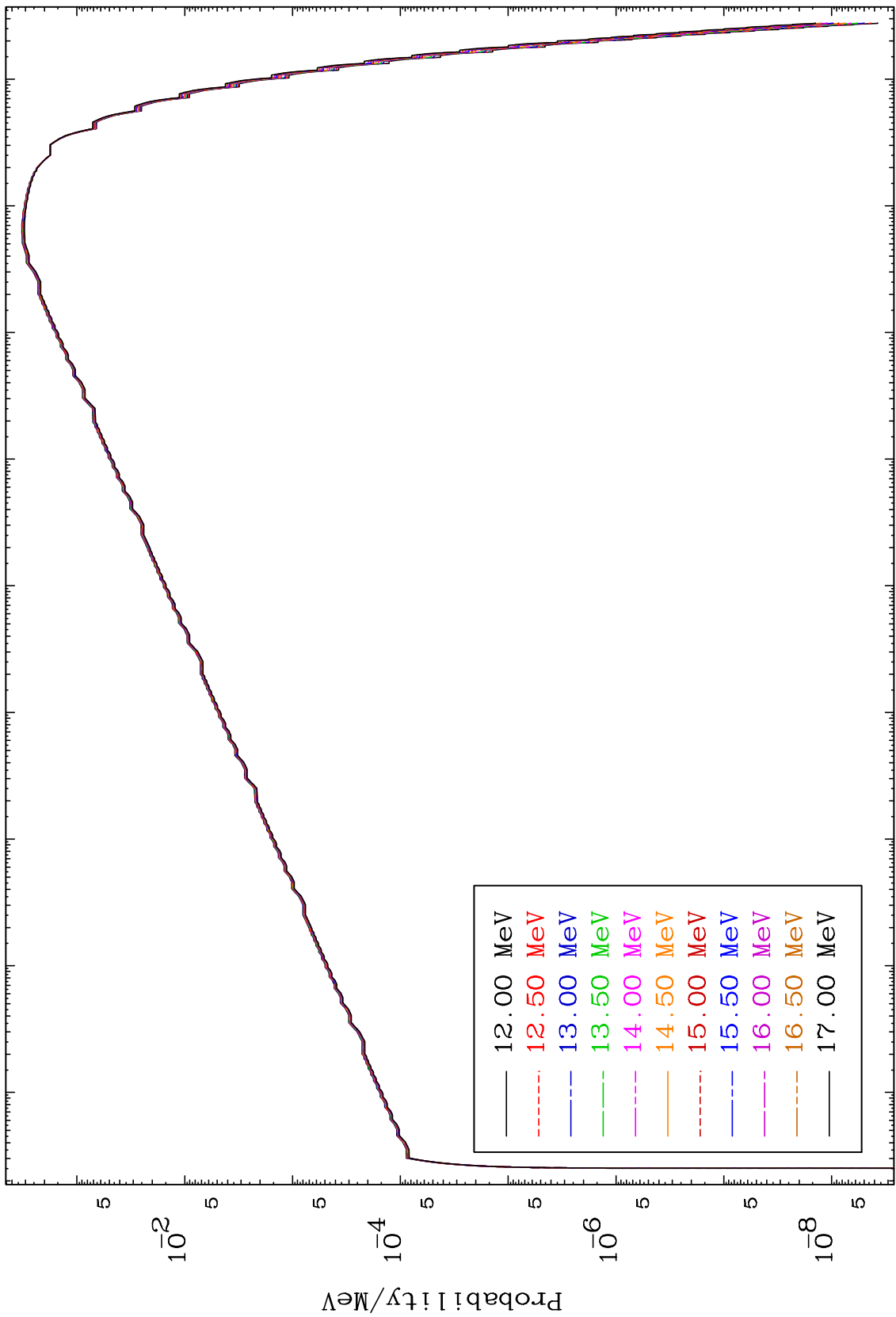
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Secondary Energy (MeV)

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Fission  
Energy Distribution

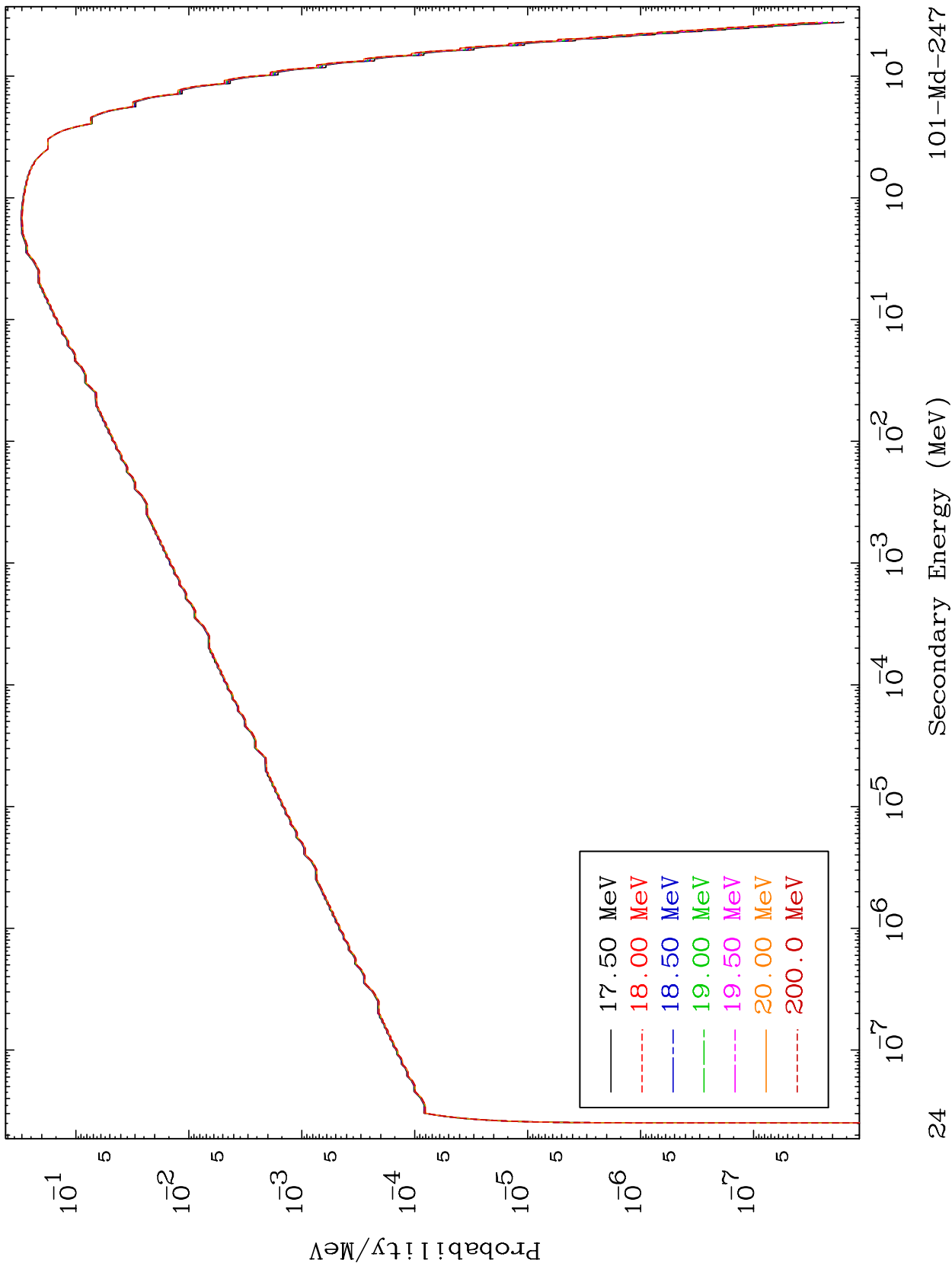
101-Md-247



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### Fission Energy Distribution

101-Md-247

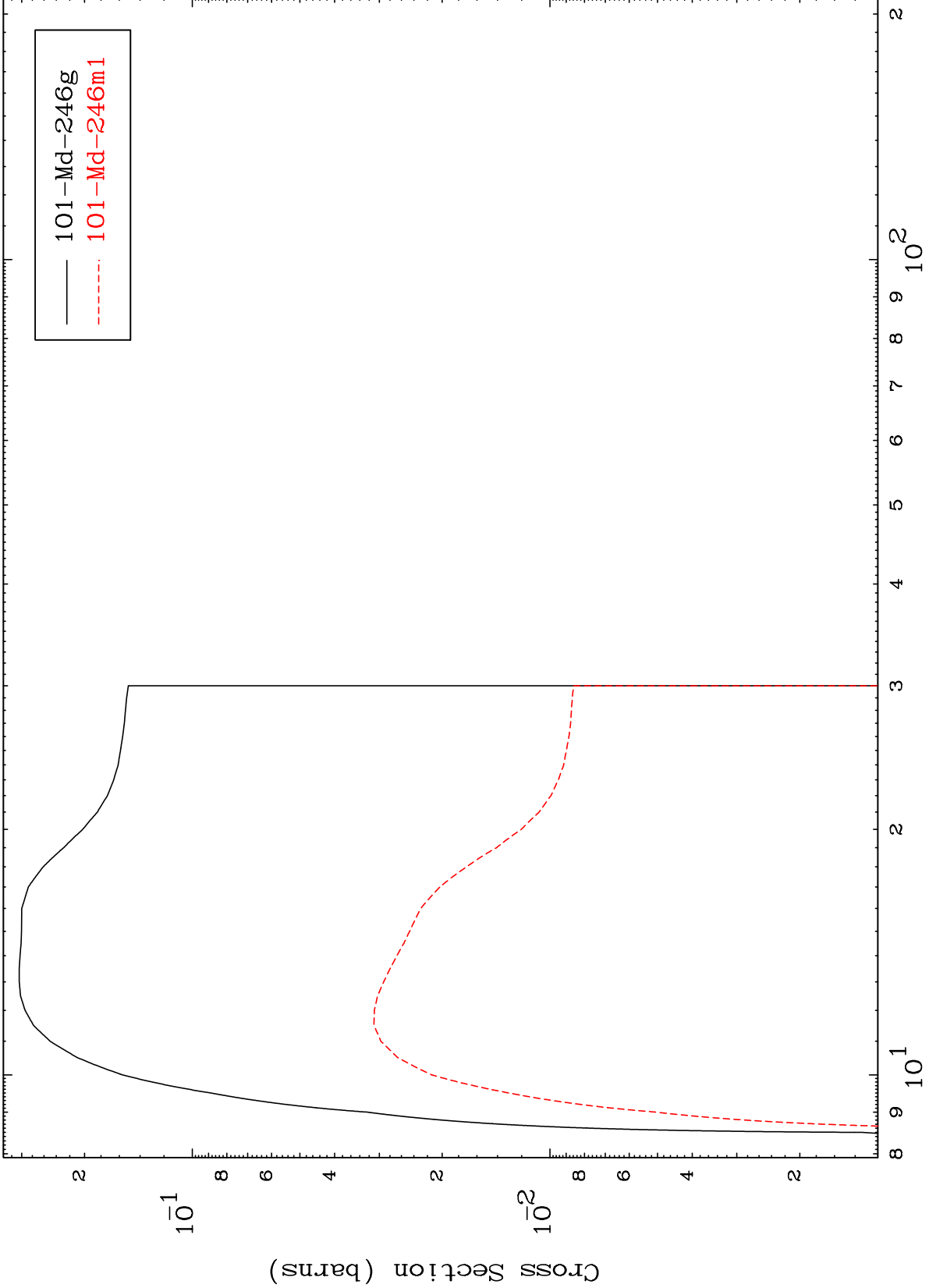


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(n,2n)

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Radionuclide Production Cross Section



25

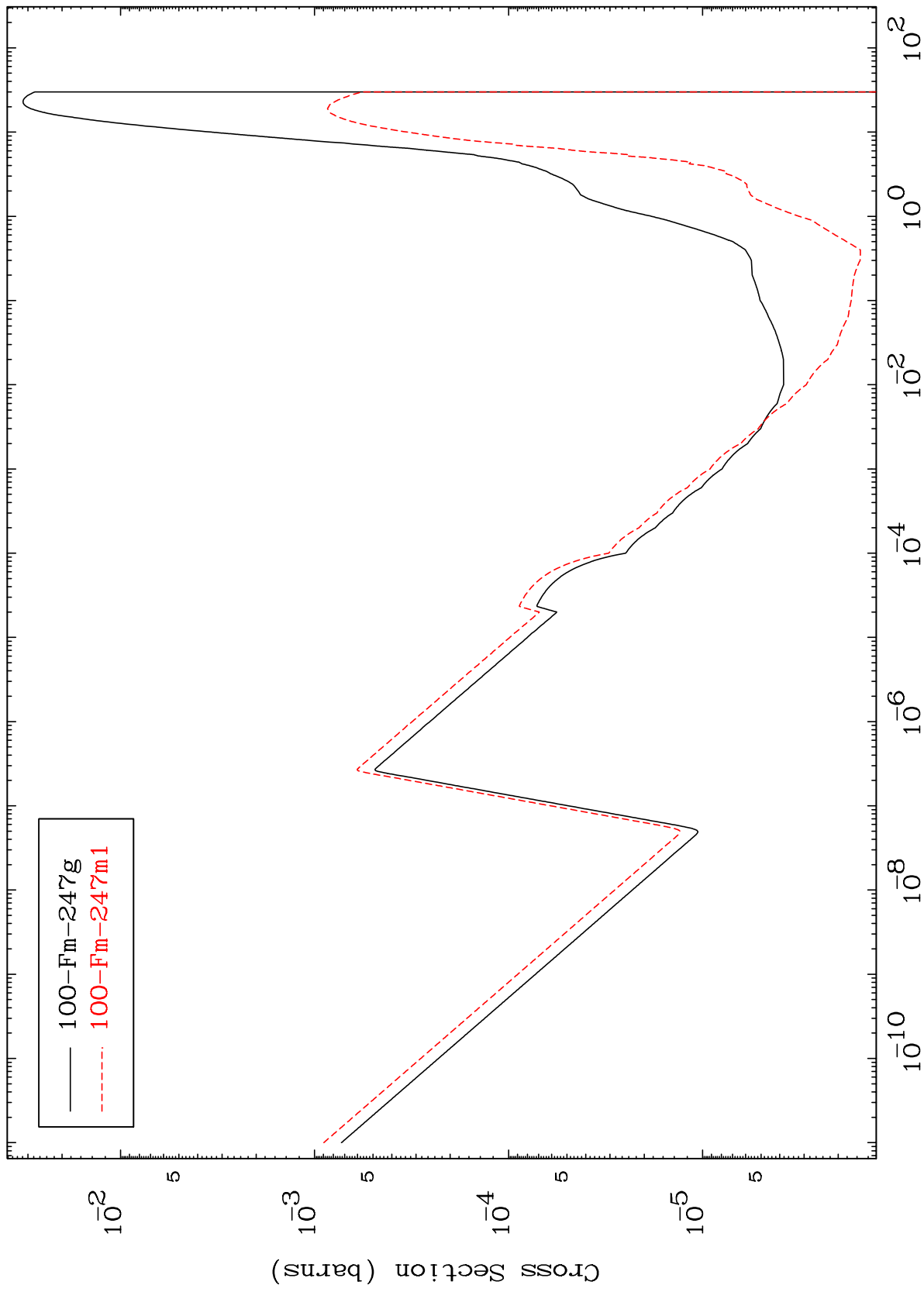
Incident Energy (MeV)

101-Md-247

MAT 9948

101-Md-247

(n,p)  
Radionuclide Production Cross Section



26

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Incident Energy (MeV)