

Program EVALPLOT
(Version 2017-1)

by

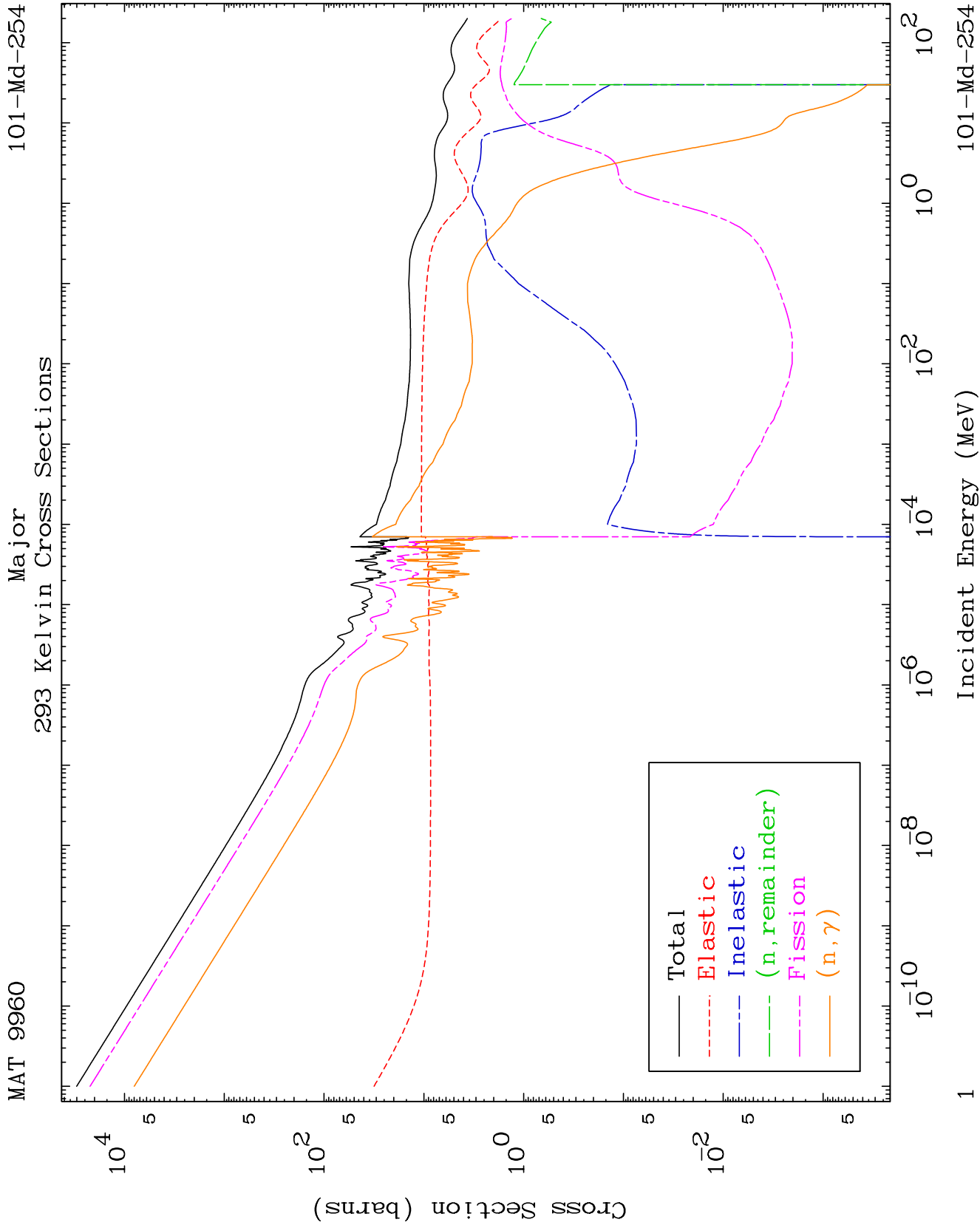
Dermott E. Cullen
(Present Contact Information)

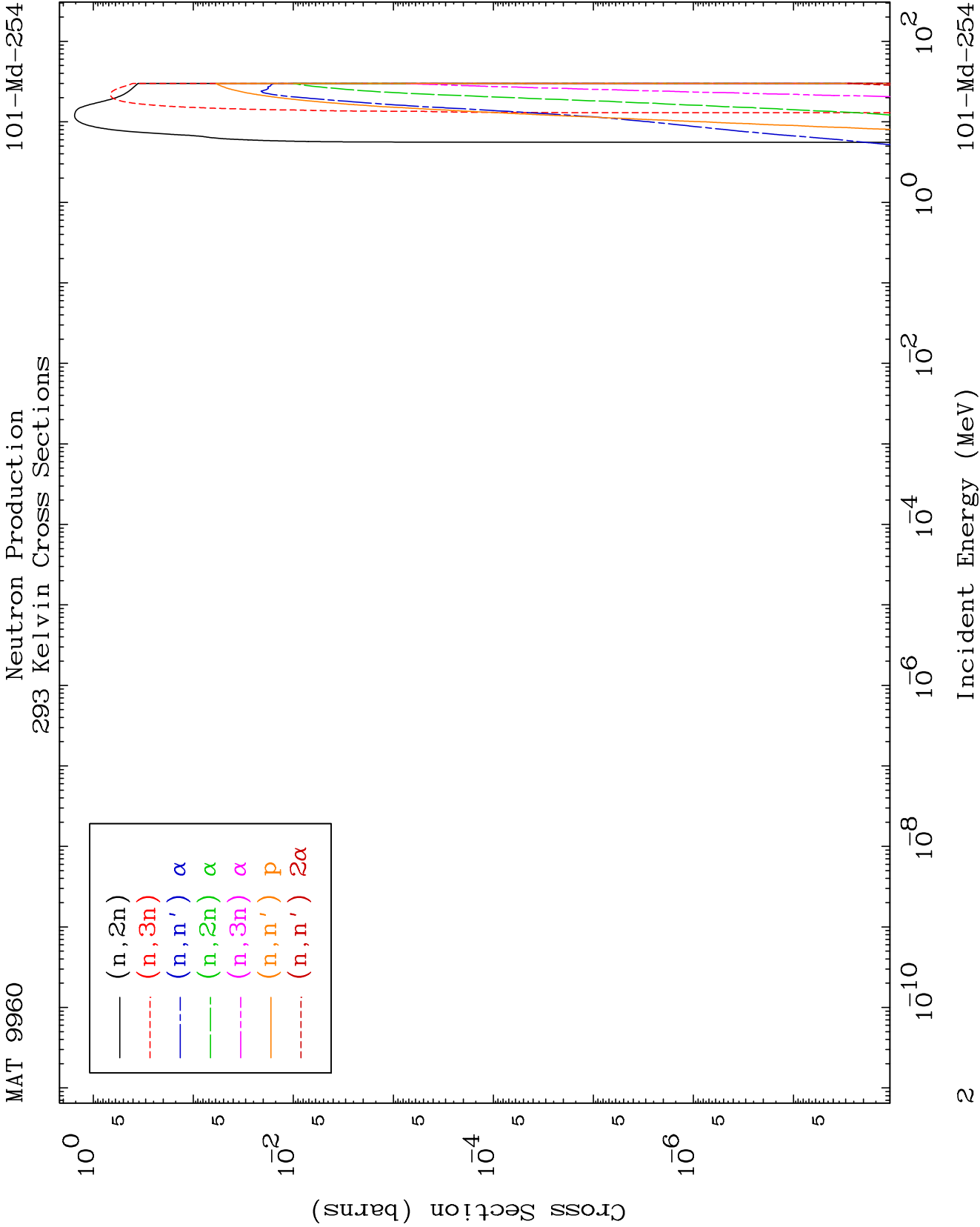
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Livermore, CA 94550
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Press Mouse Button to Start

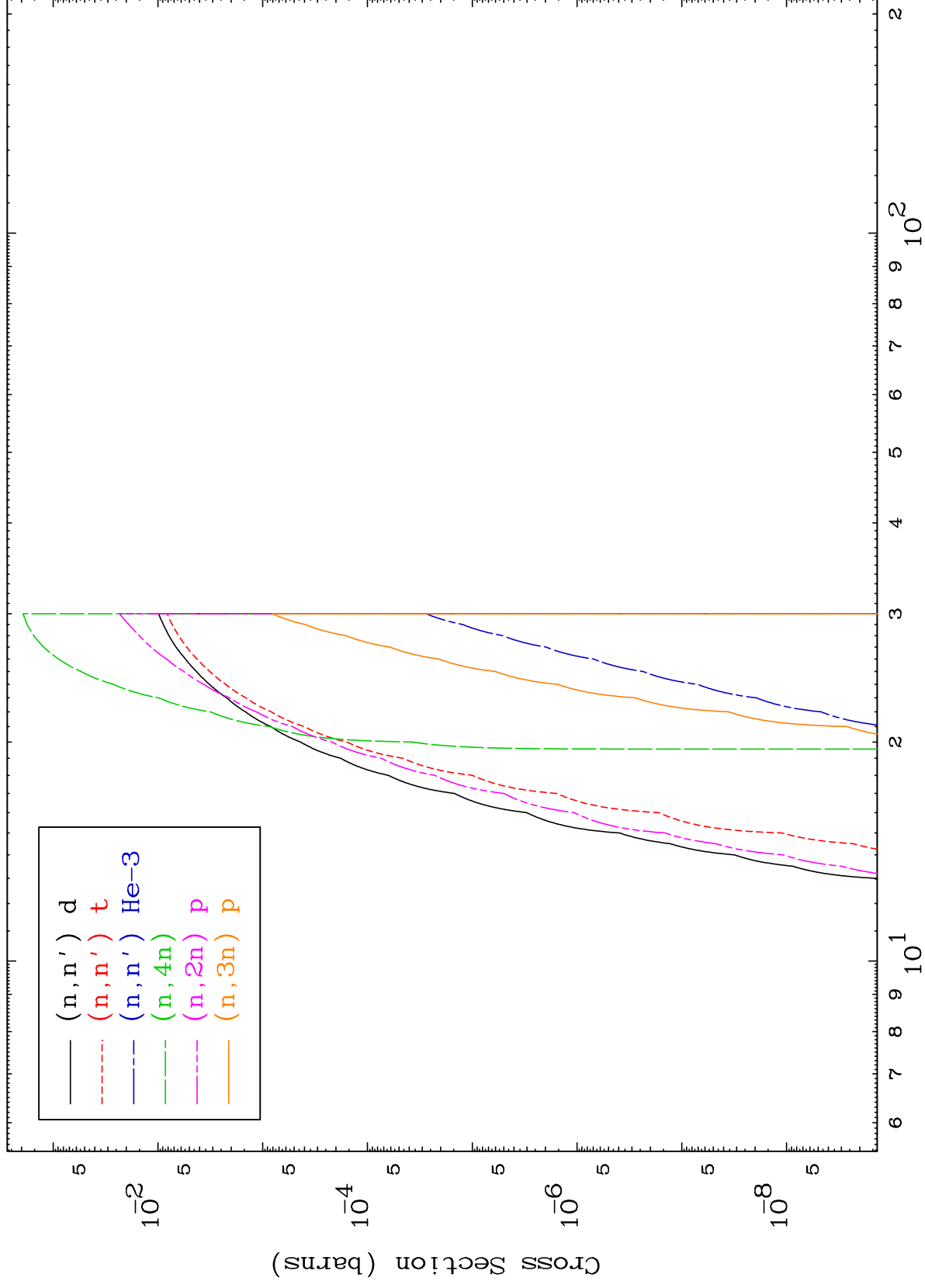




MAT 9960

Neutron Production
293 Kelvin Cross Sections

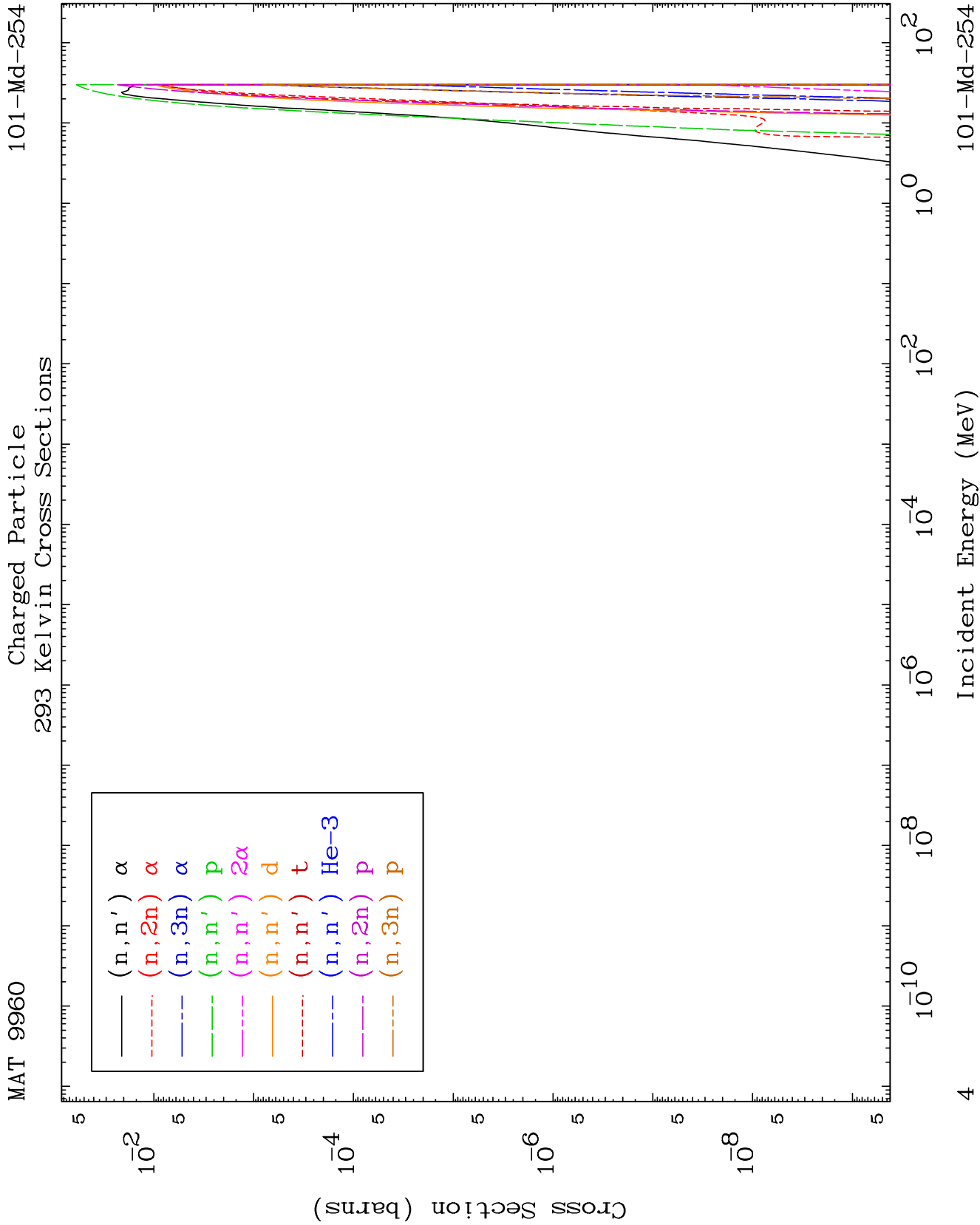
101-Md-254

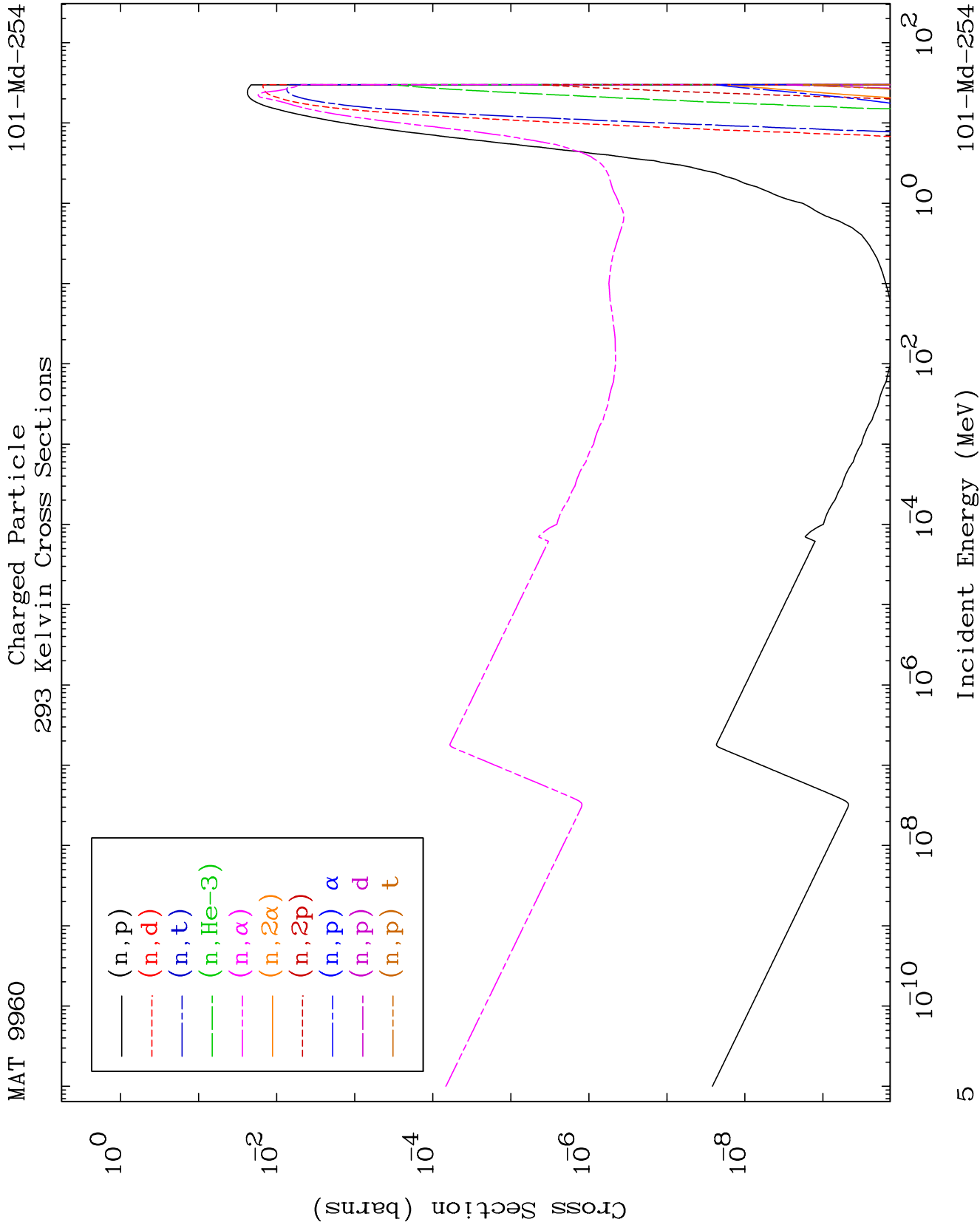


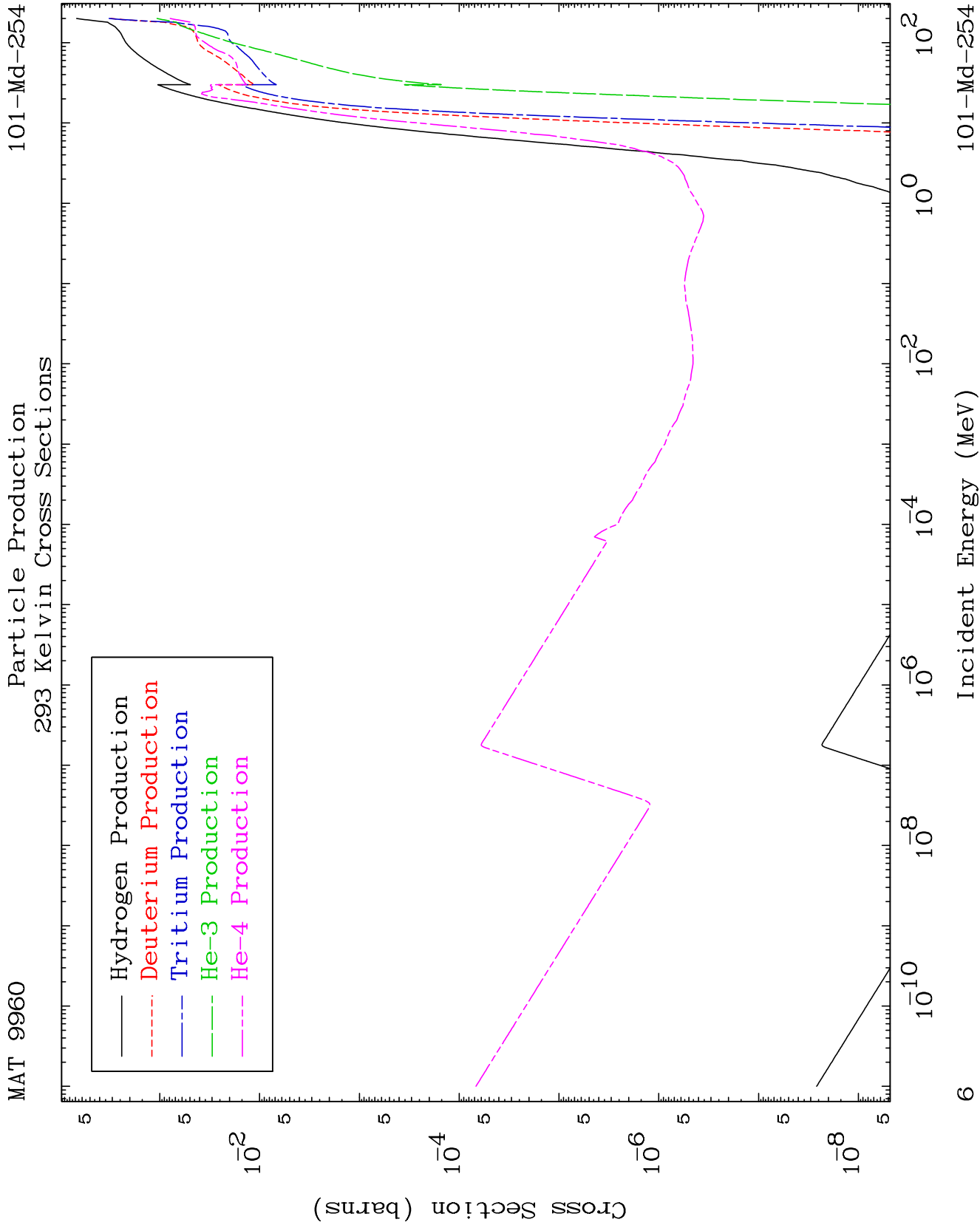
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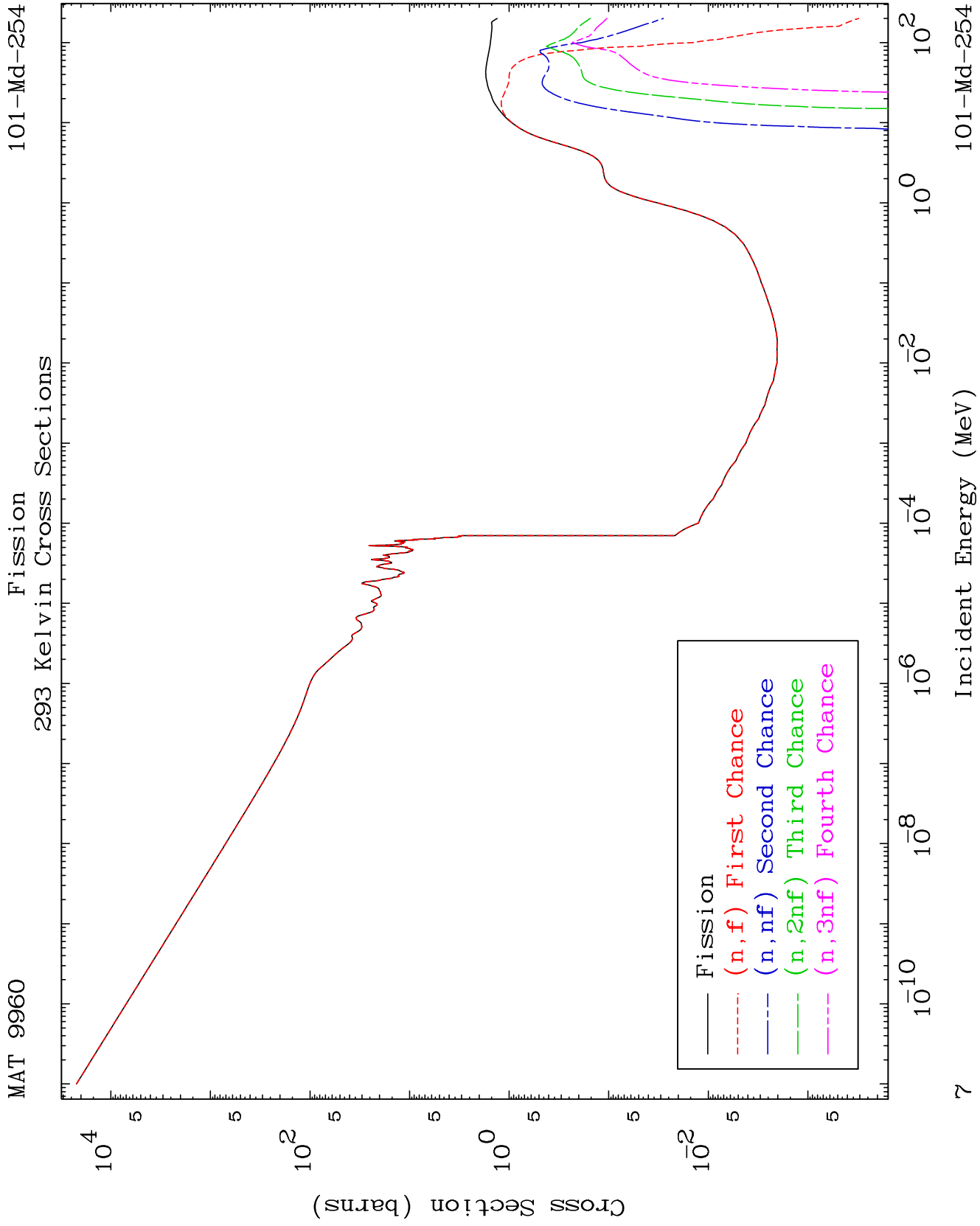
Incident Energy (MeV)

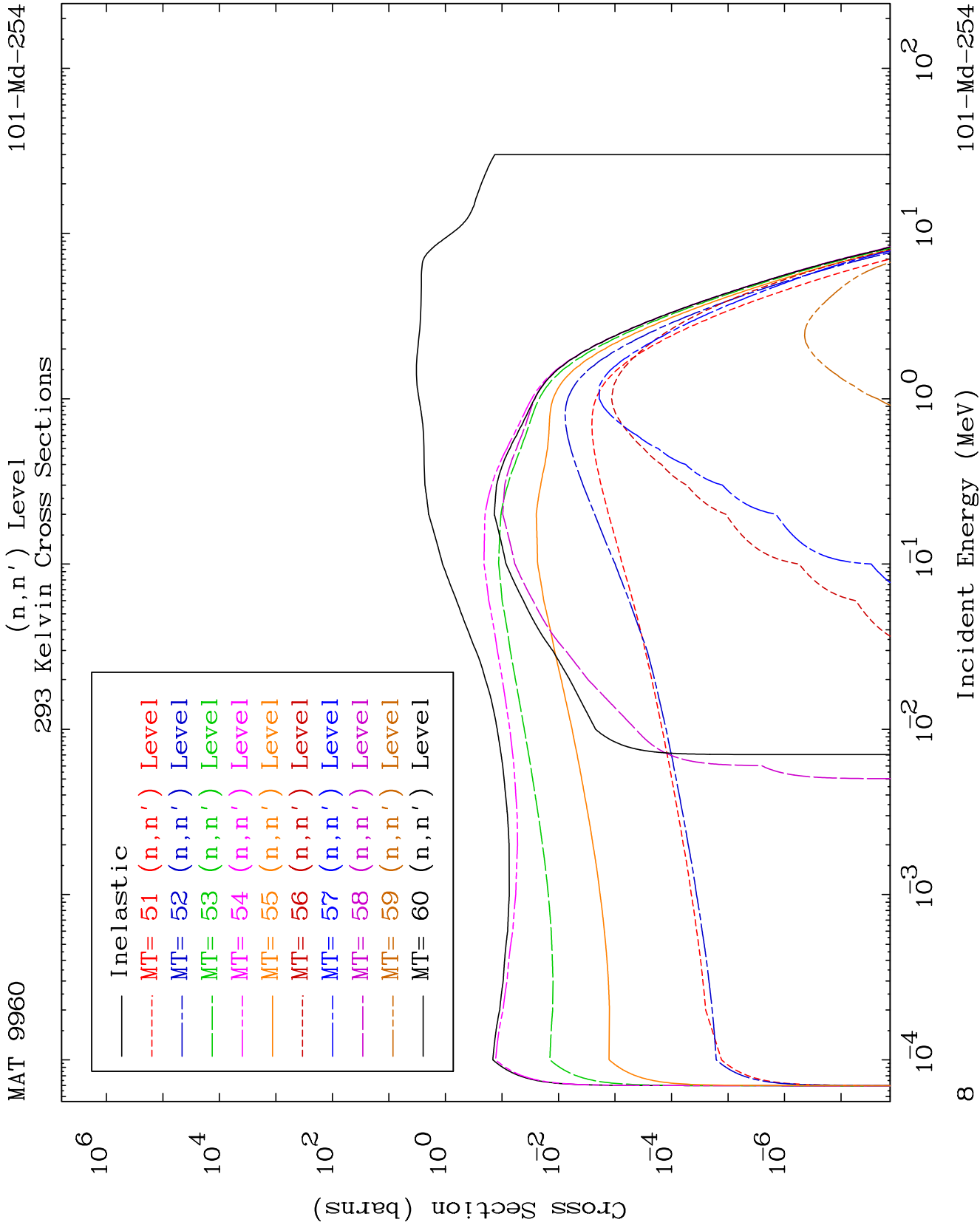
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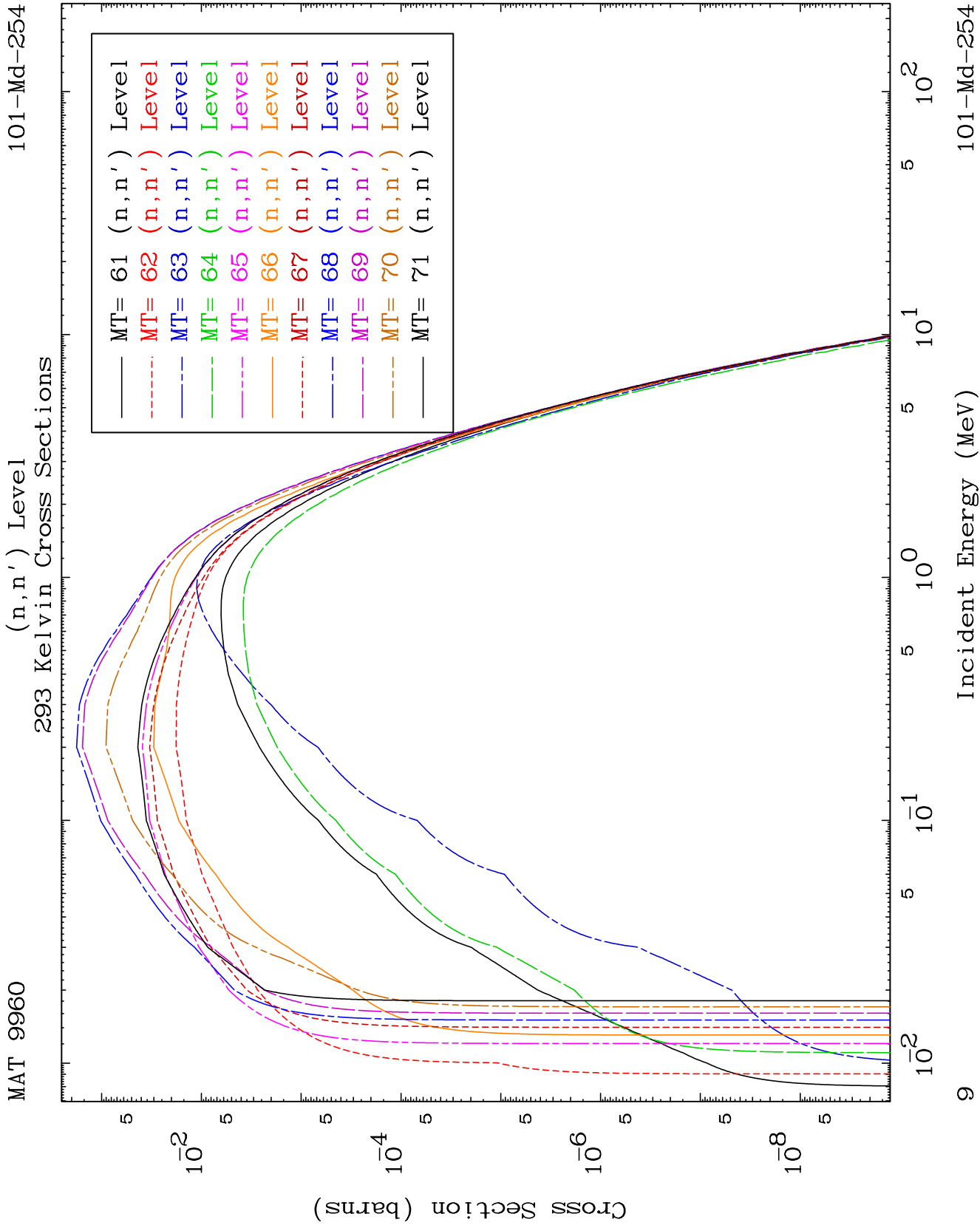


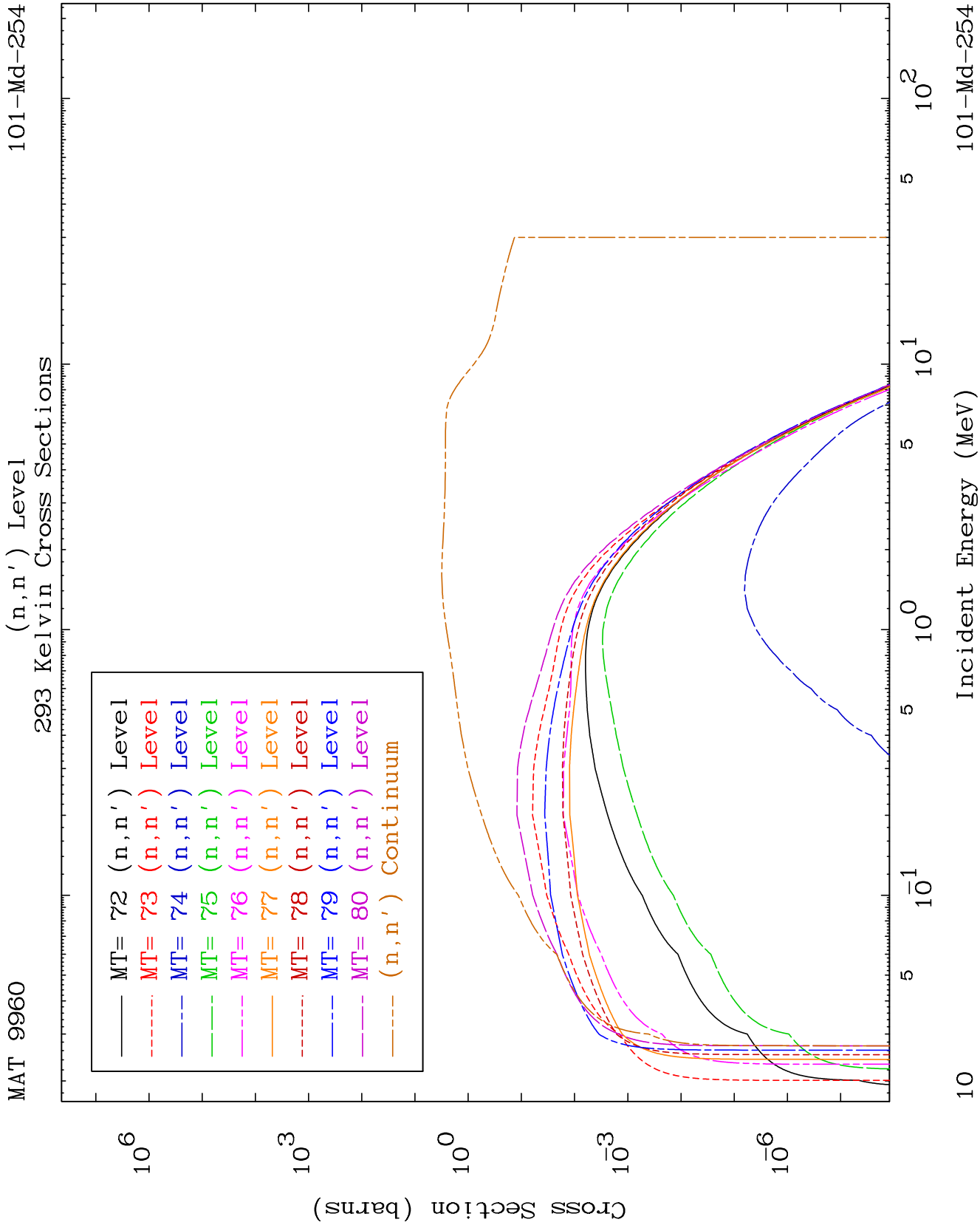








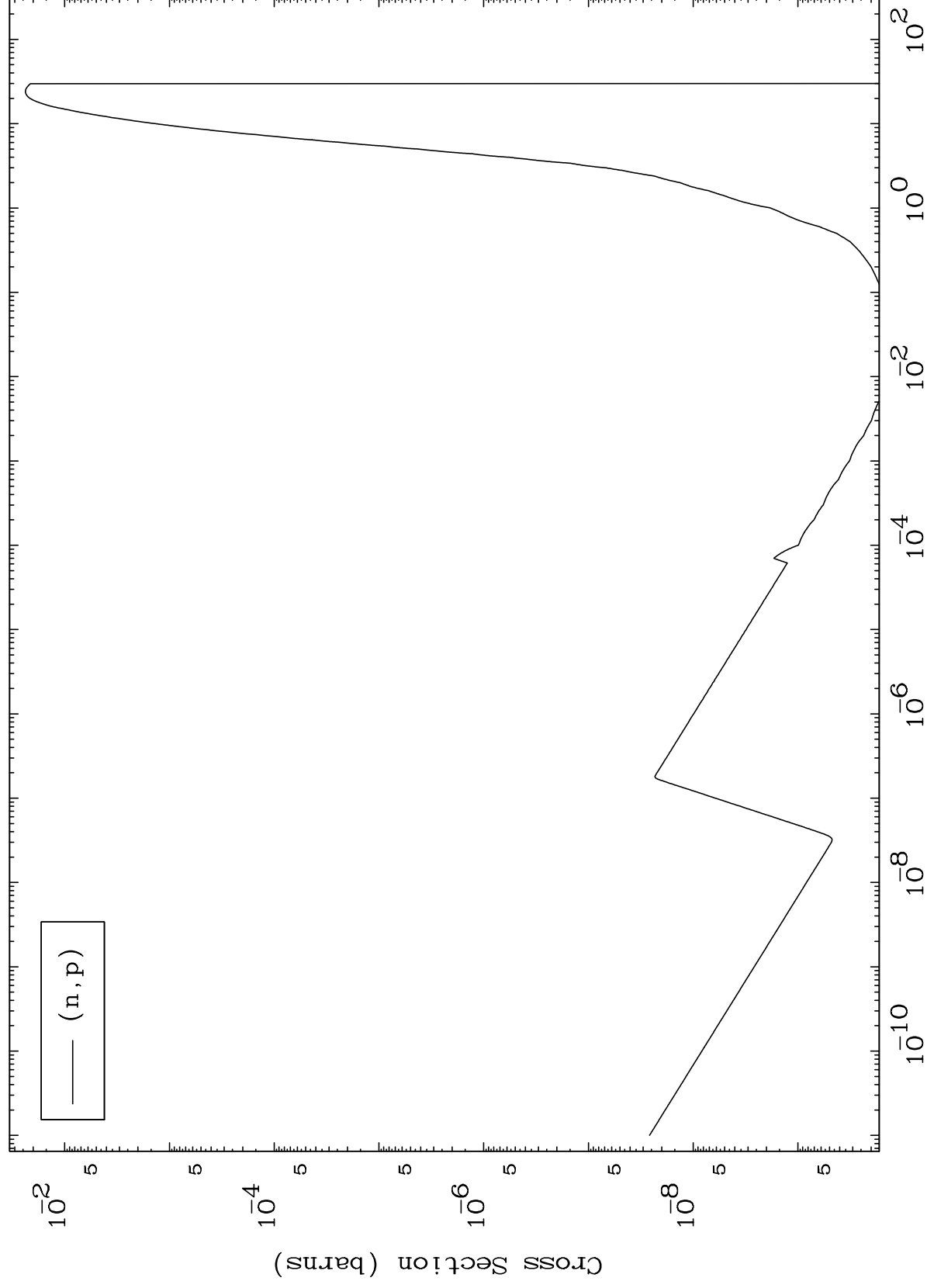


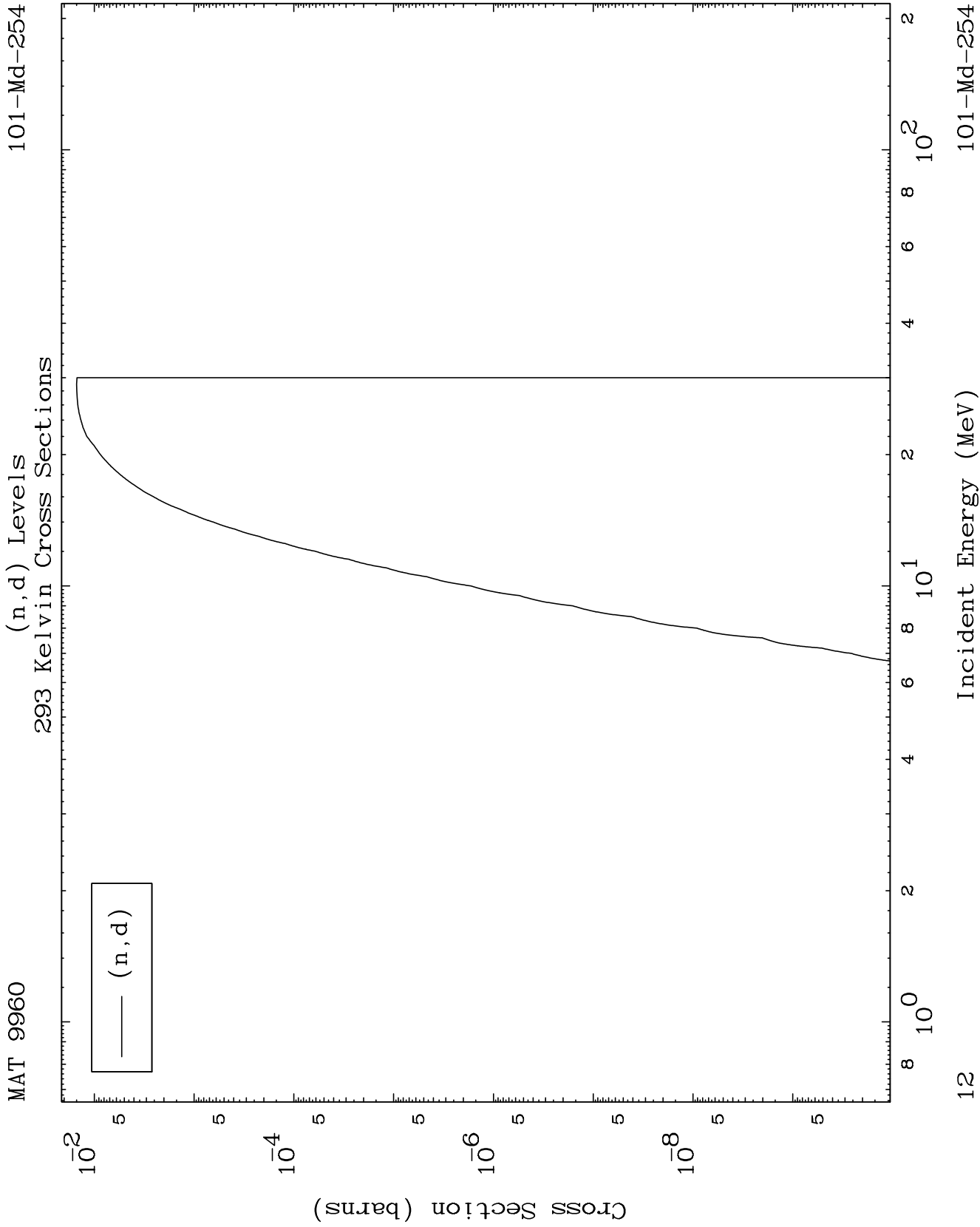


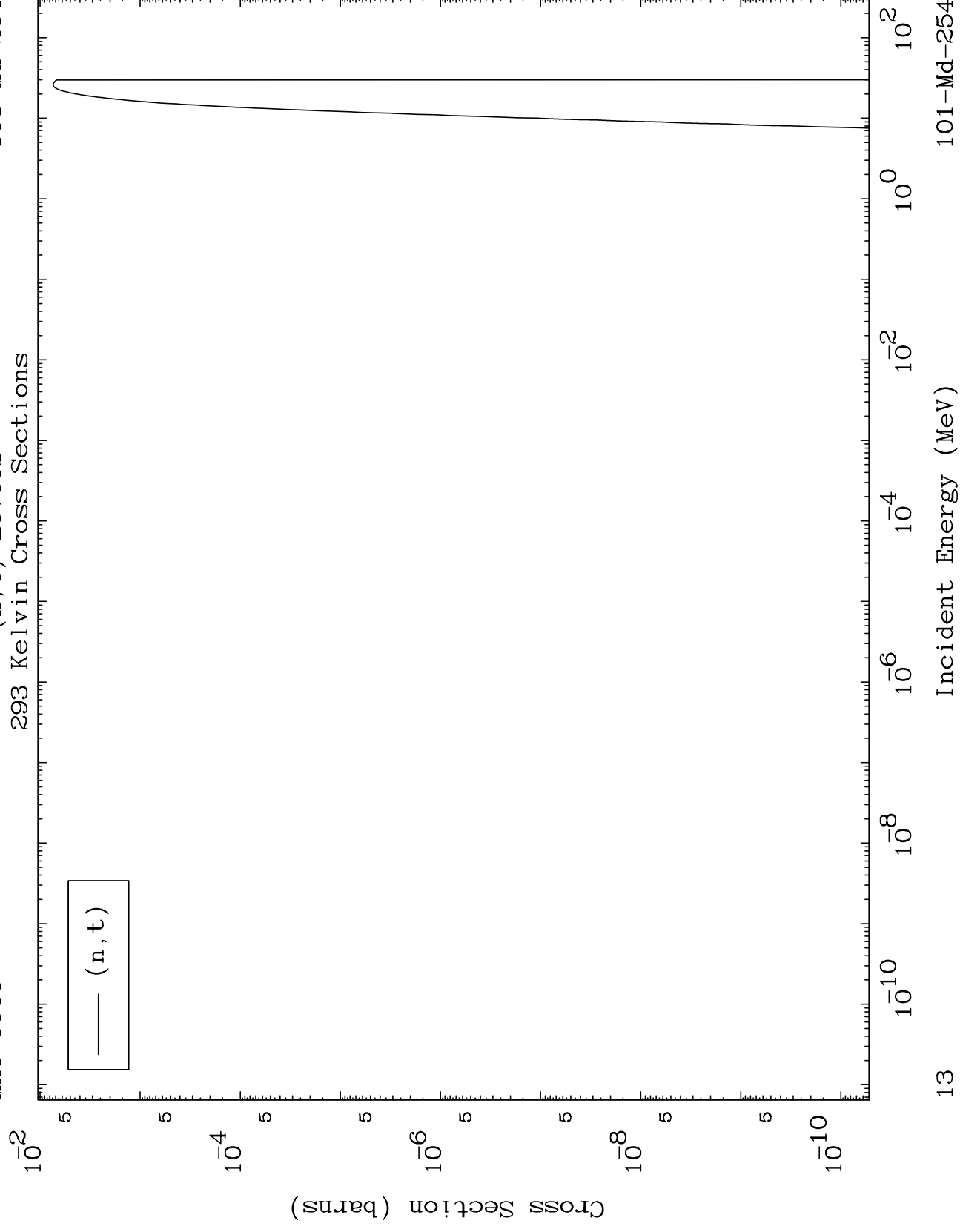
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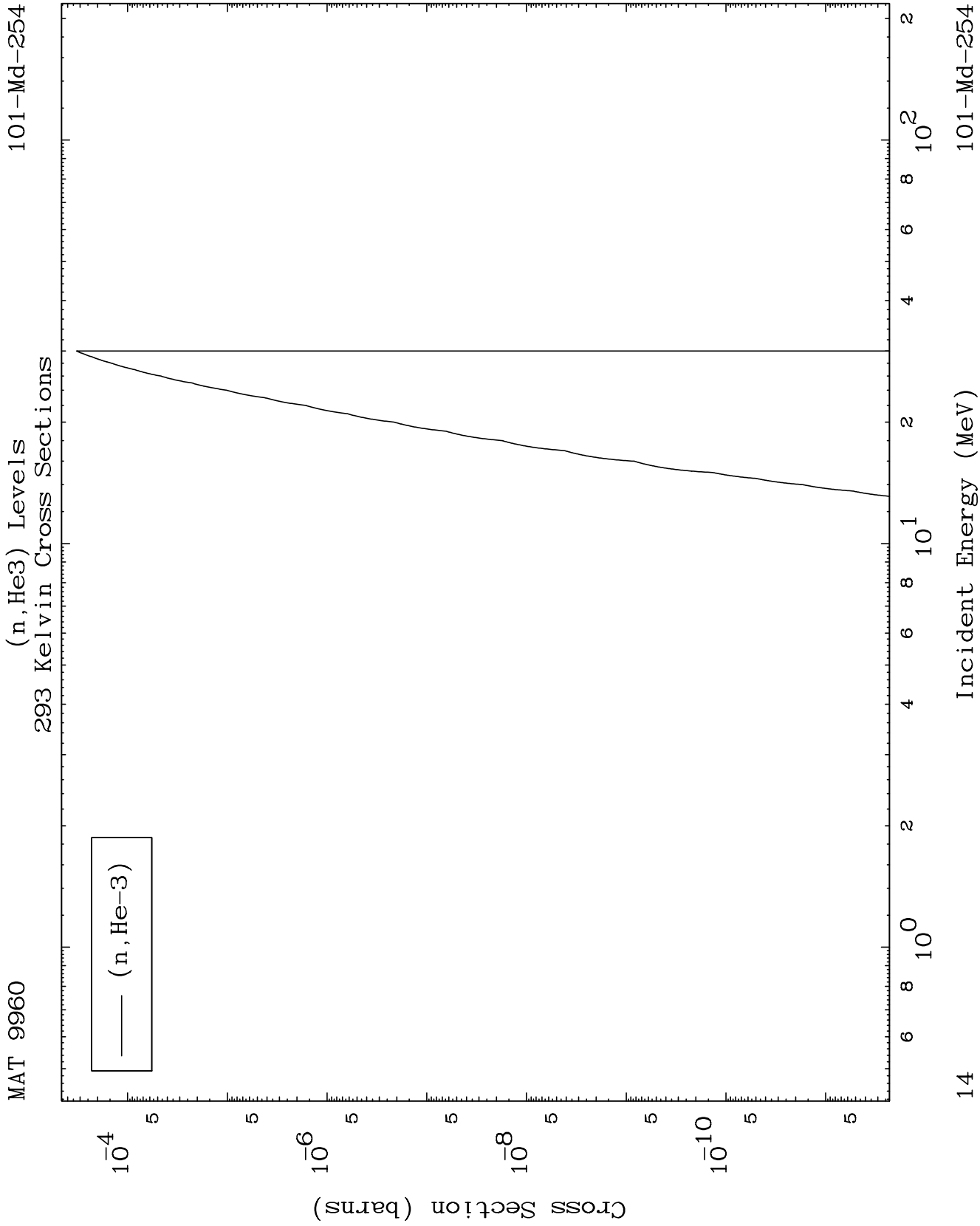
(n,p) Levels
293 Kelvin Cross Sections

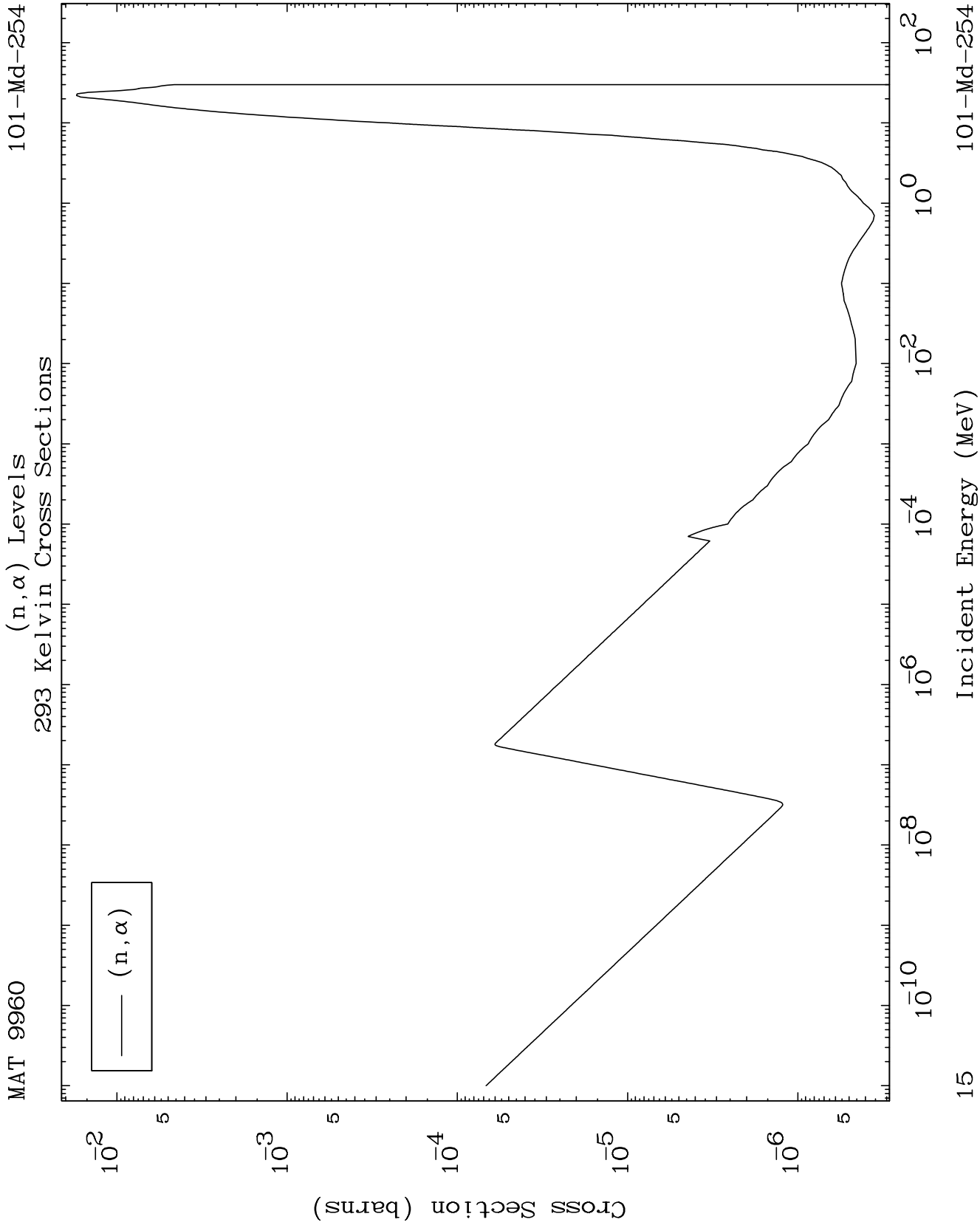
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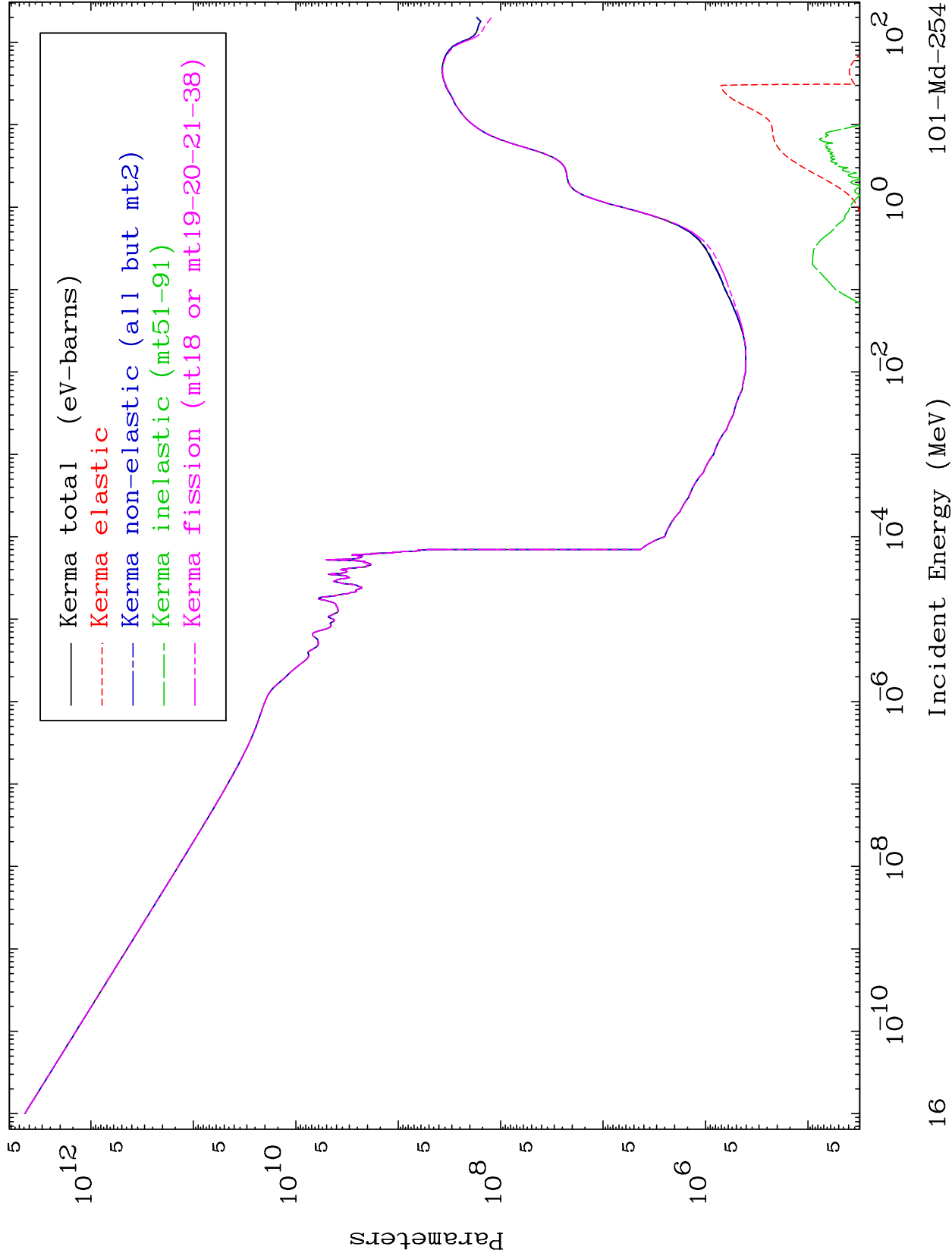


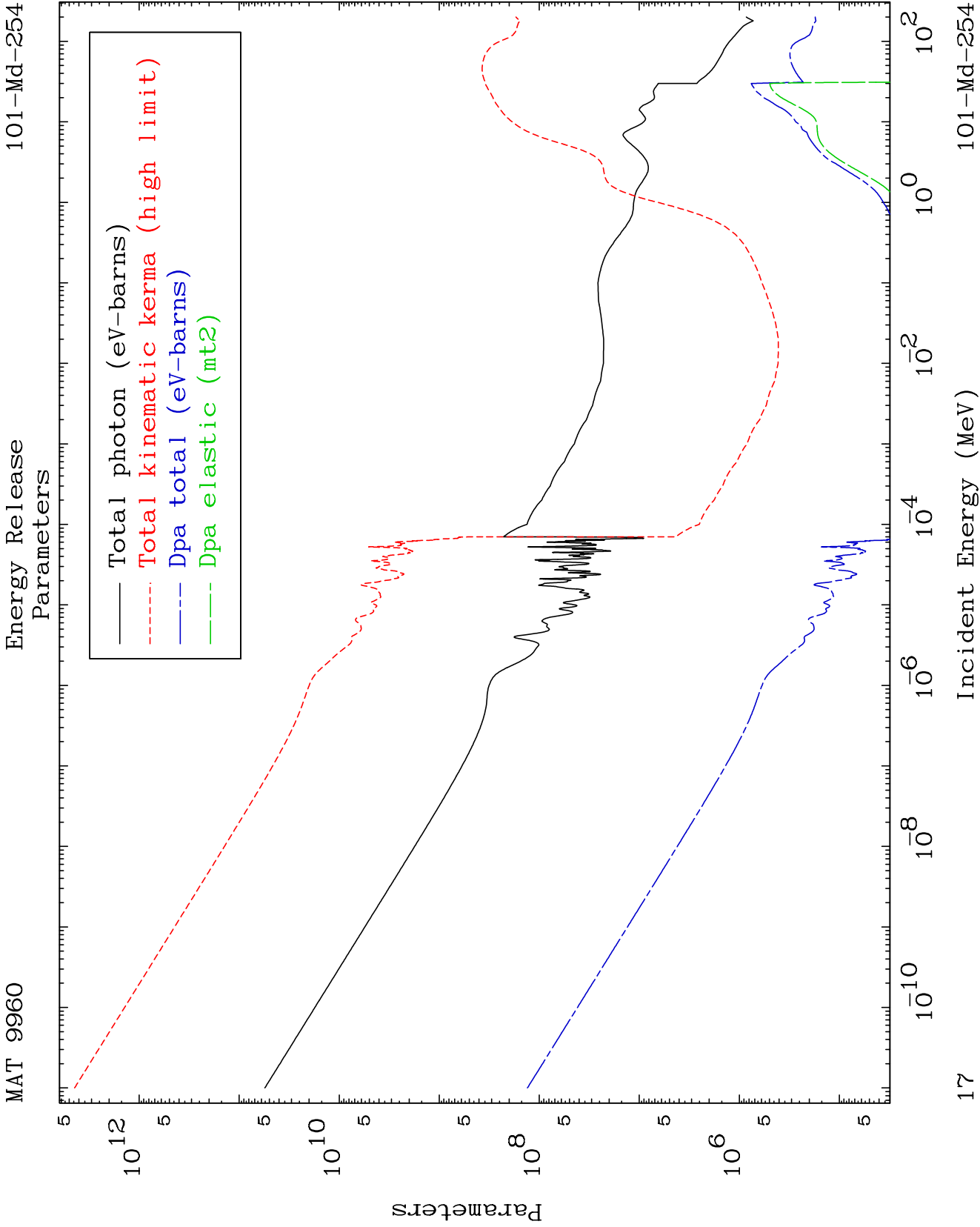


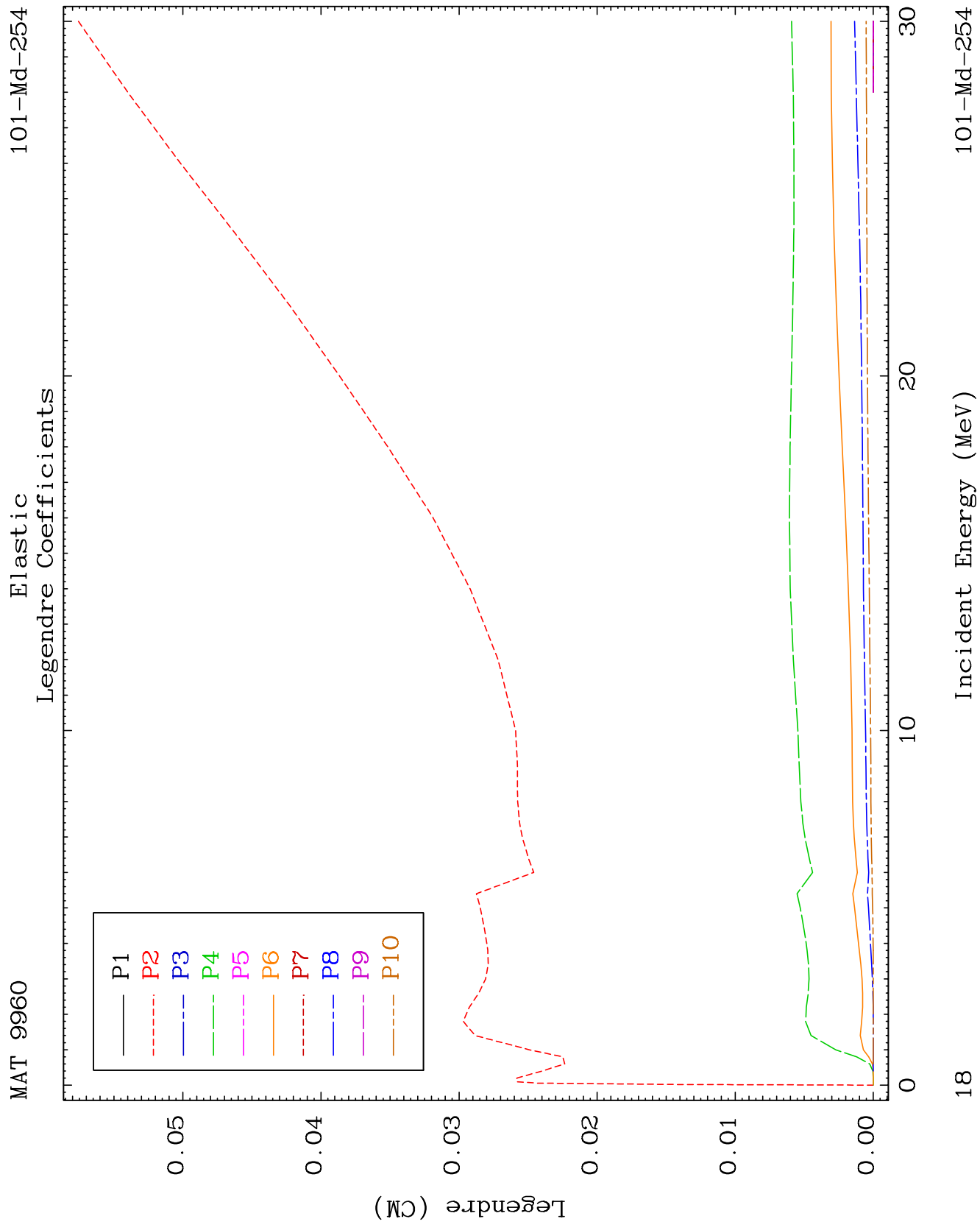


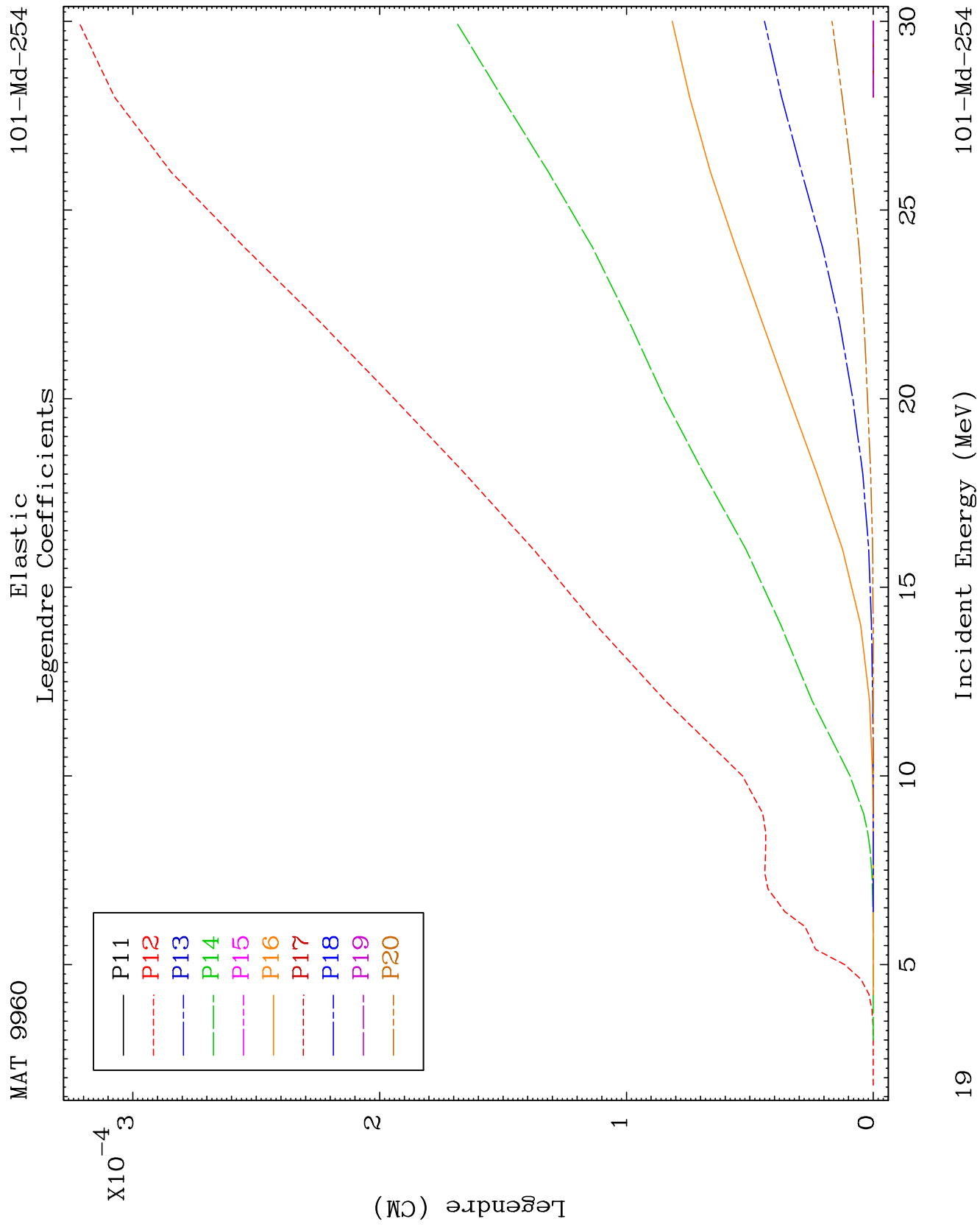


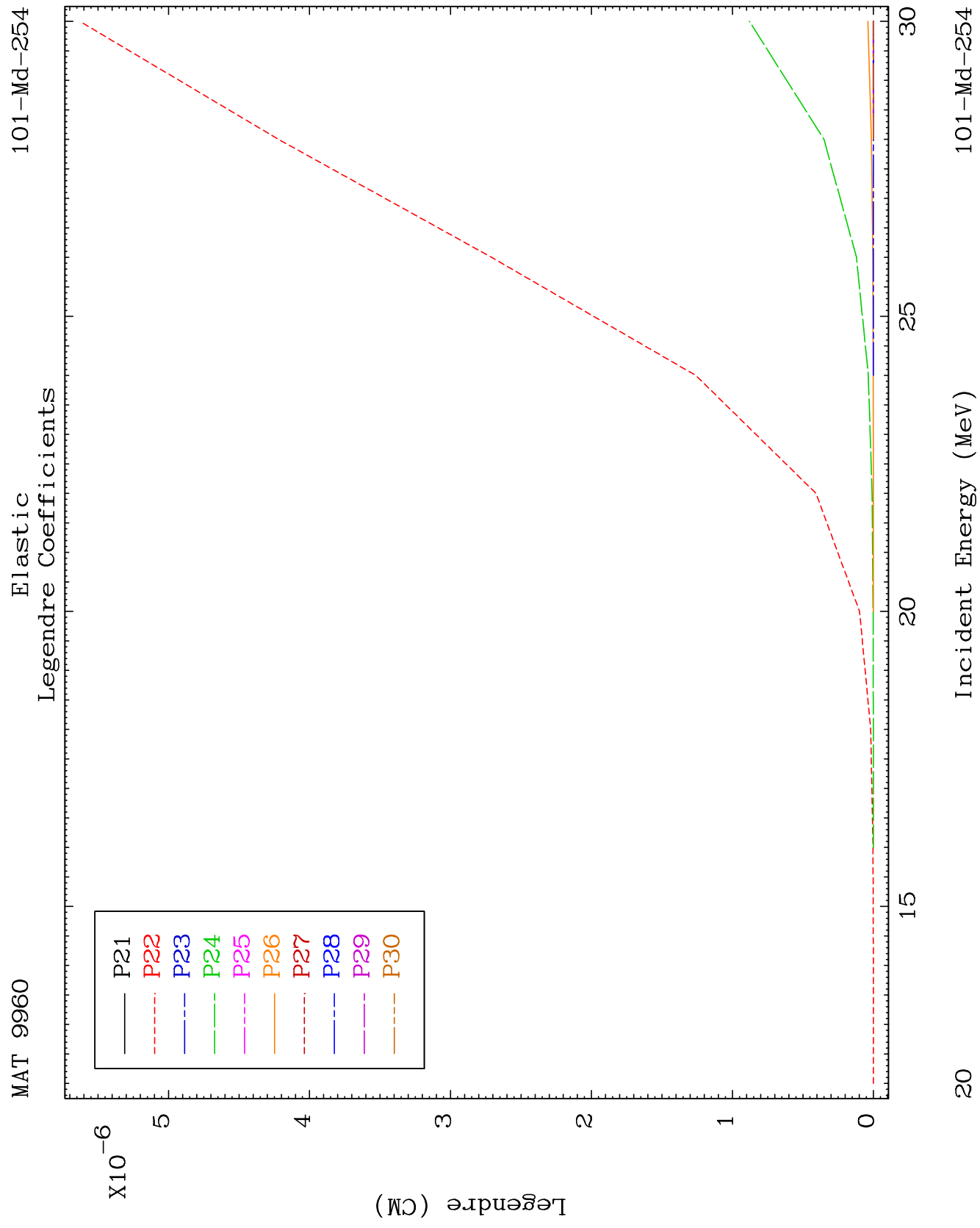








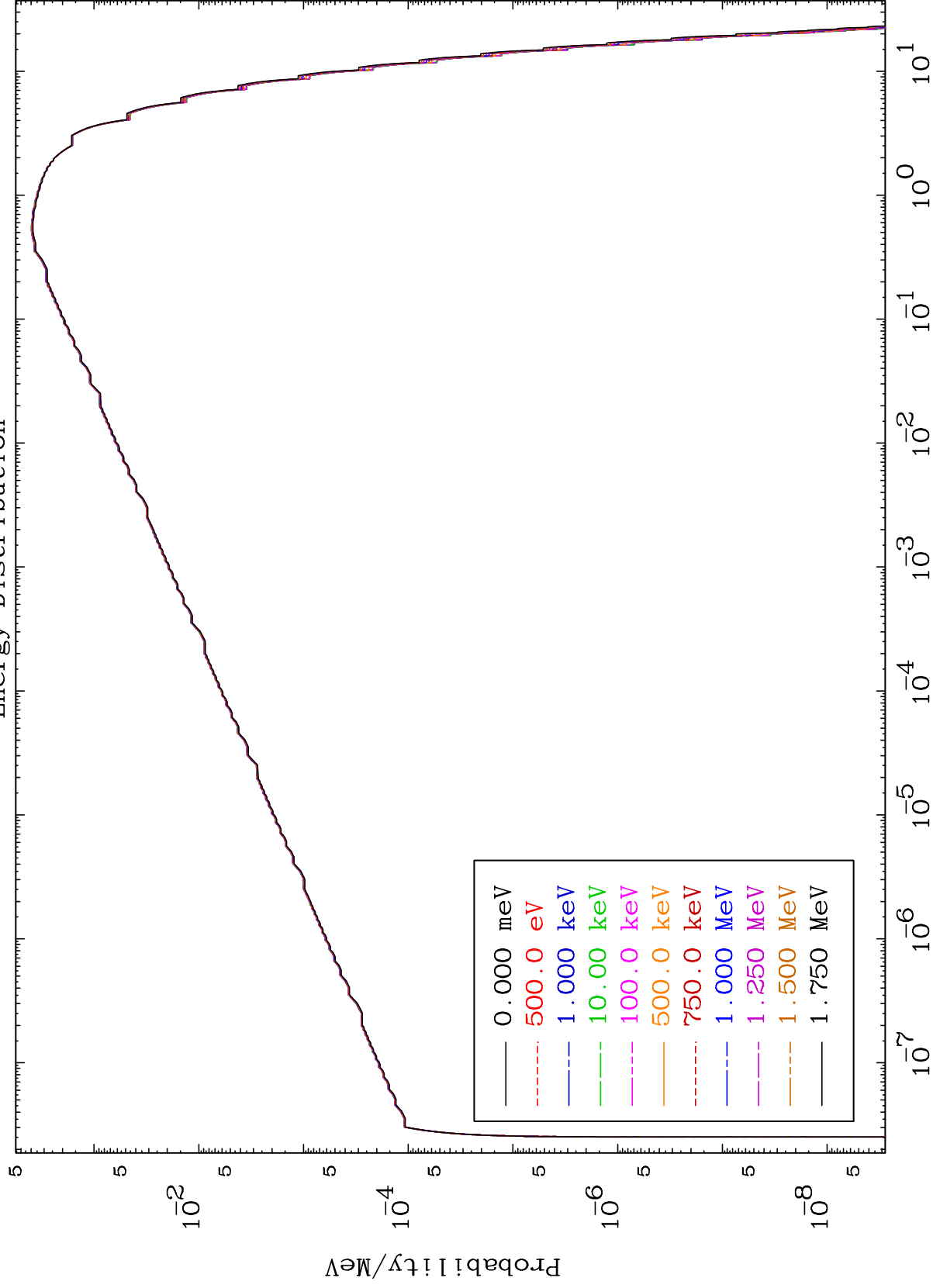


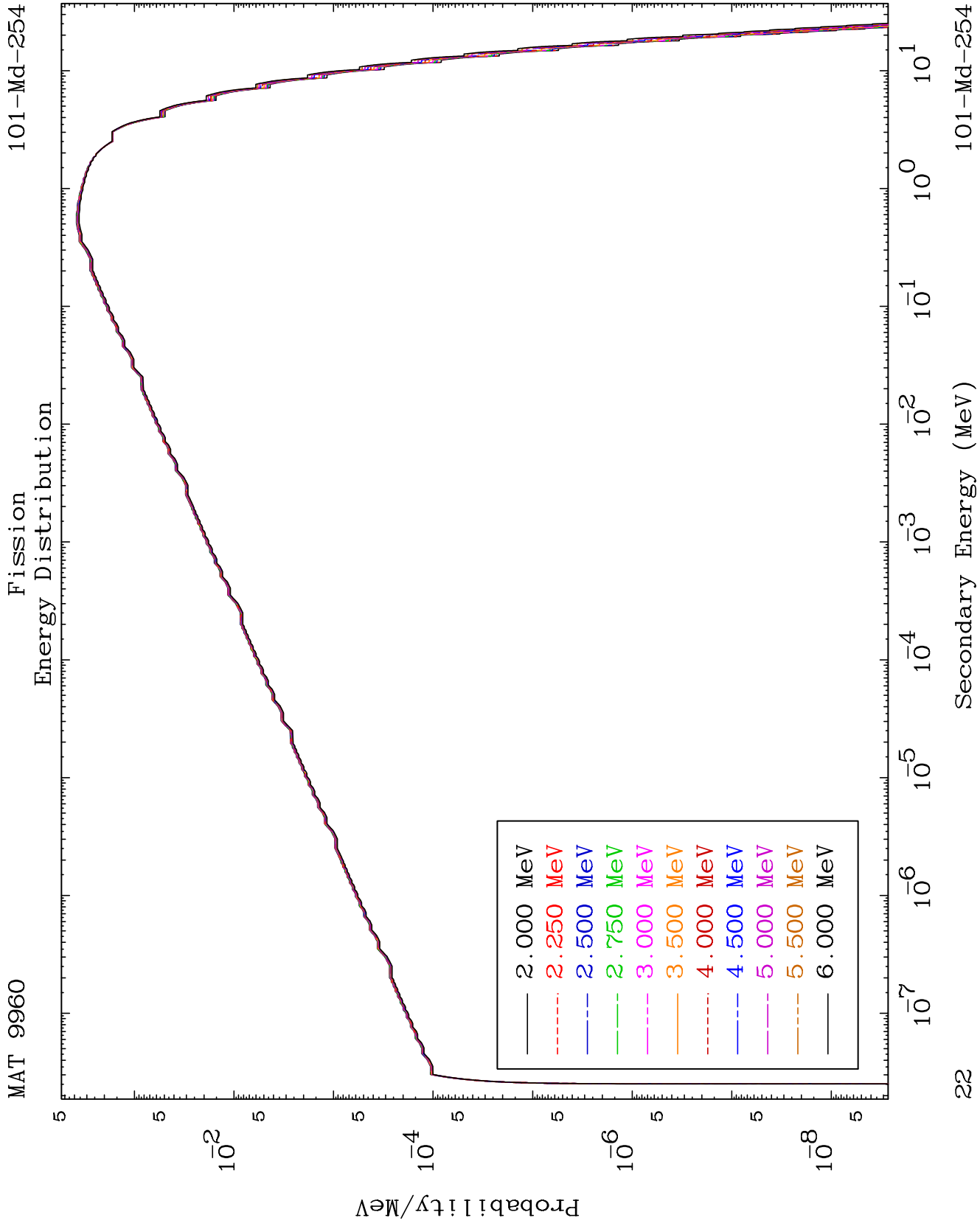


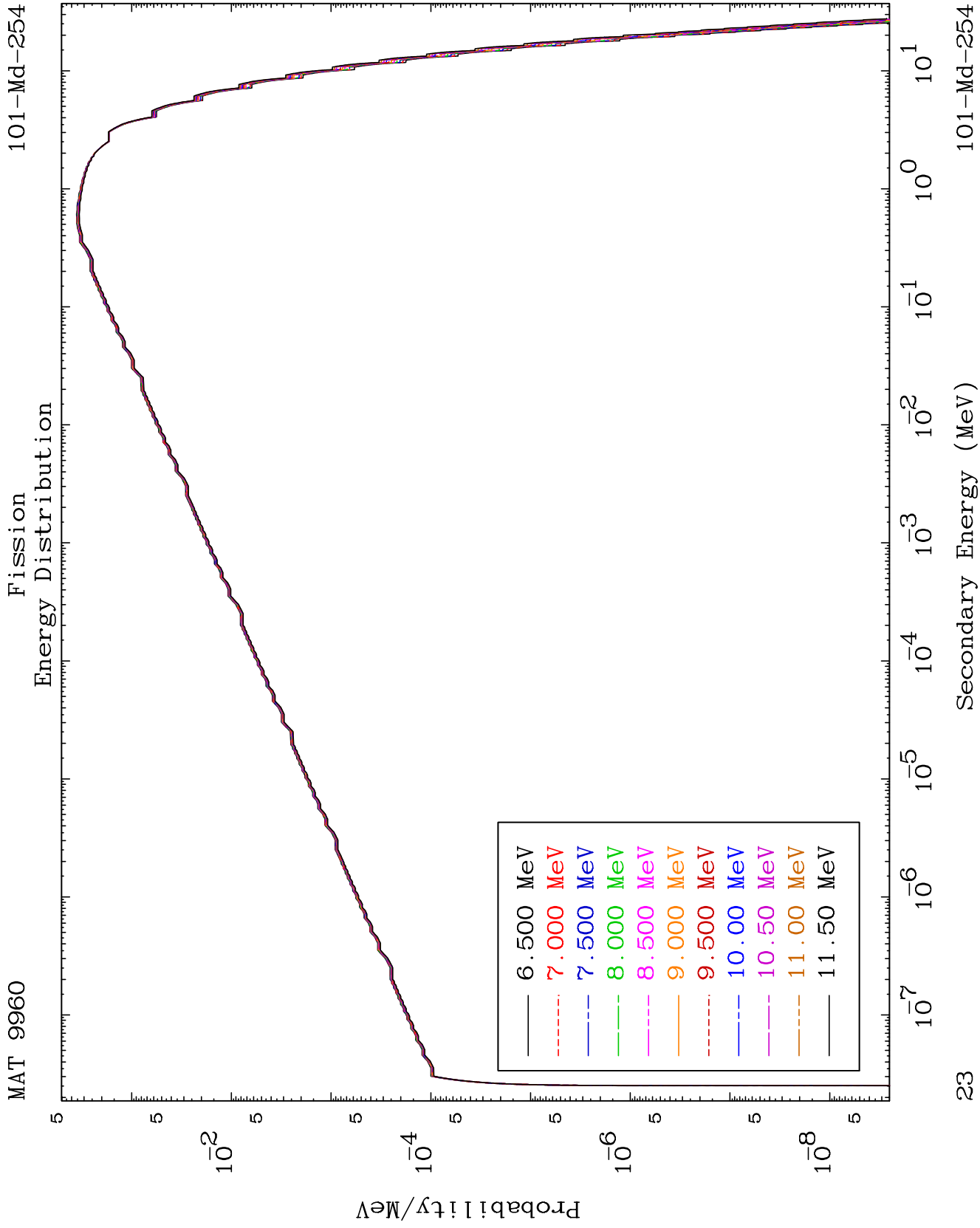
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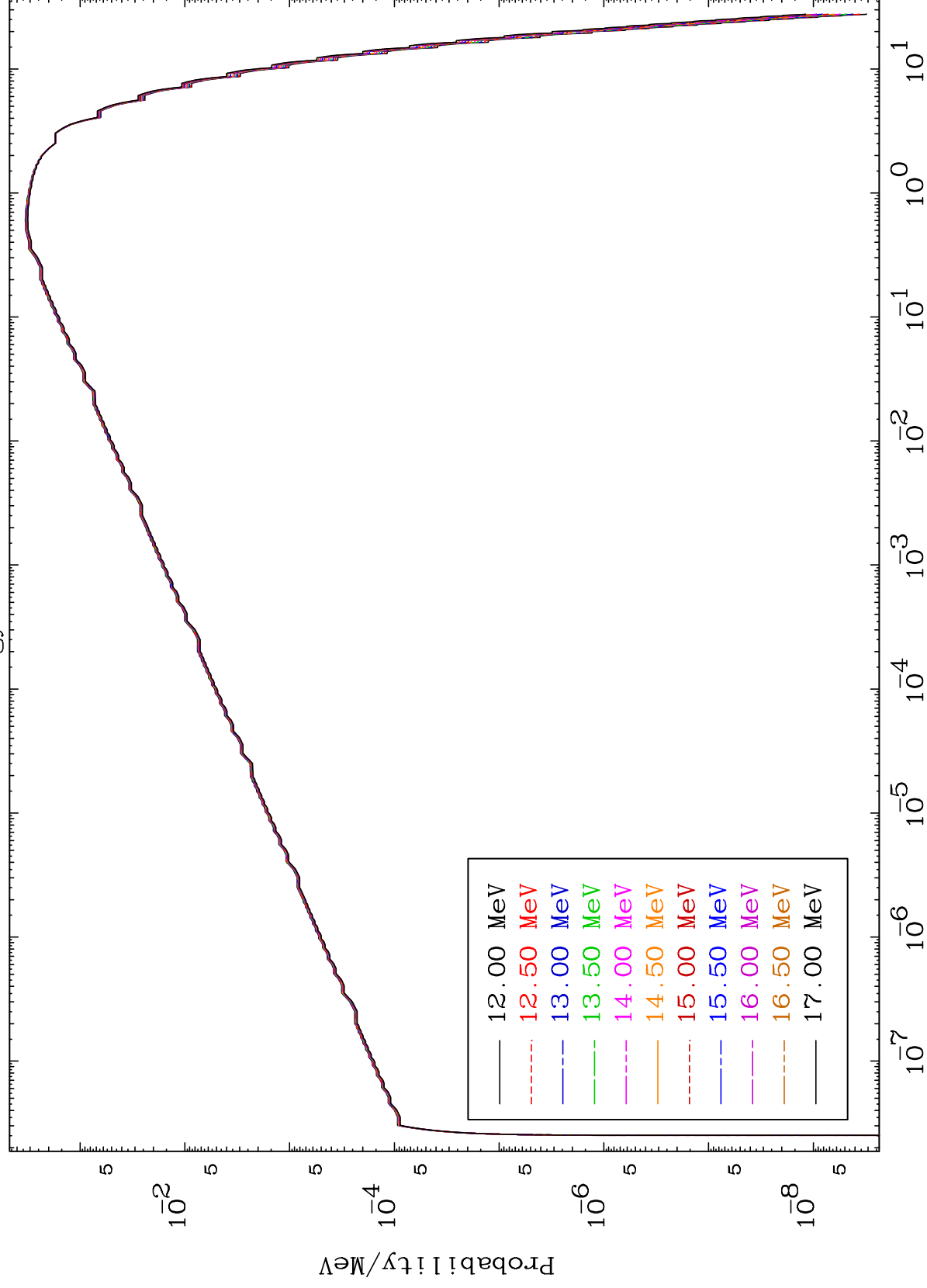
Fission
Energy Distribution

101-Md-254





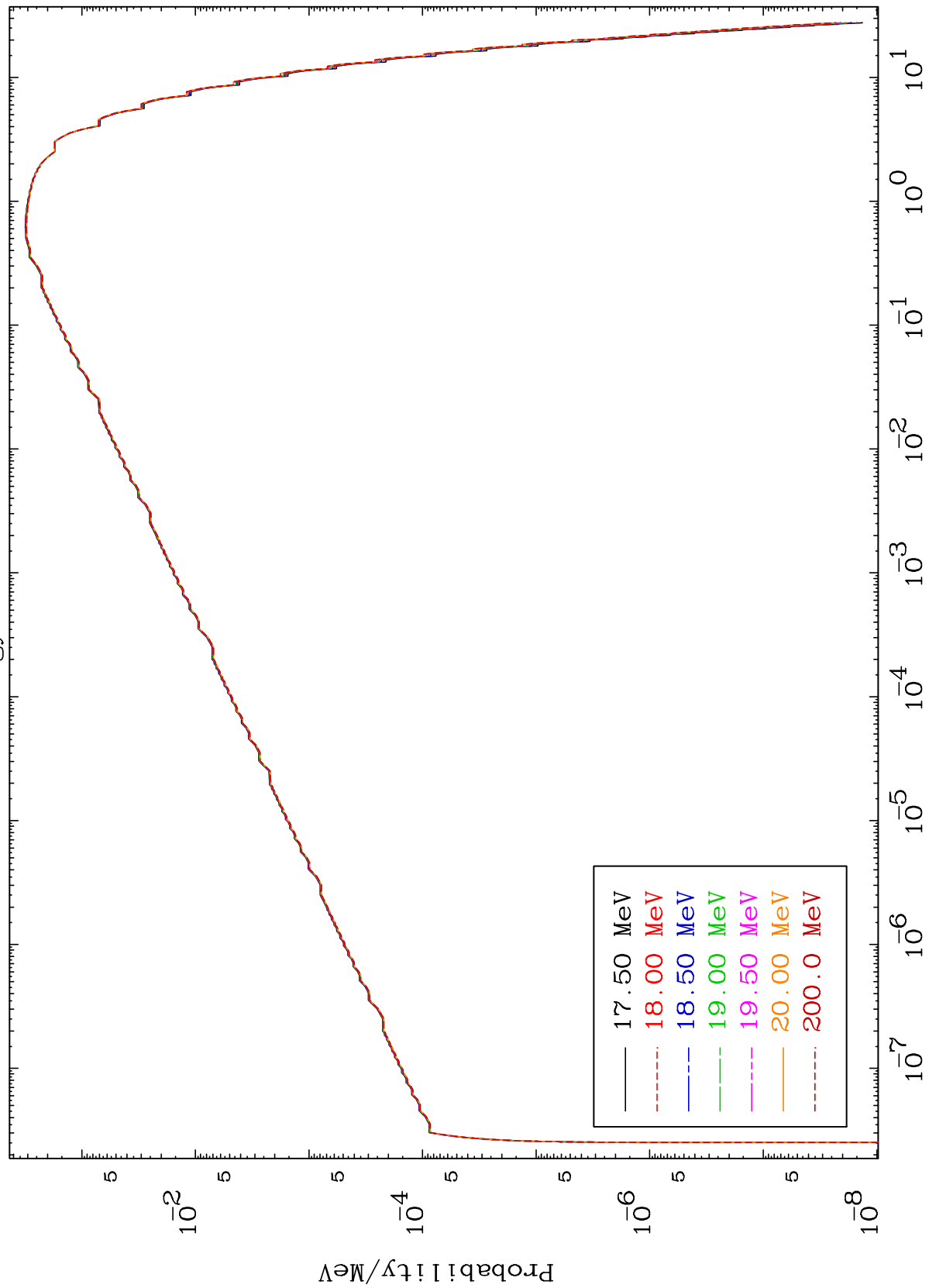


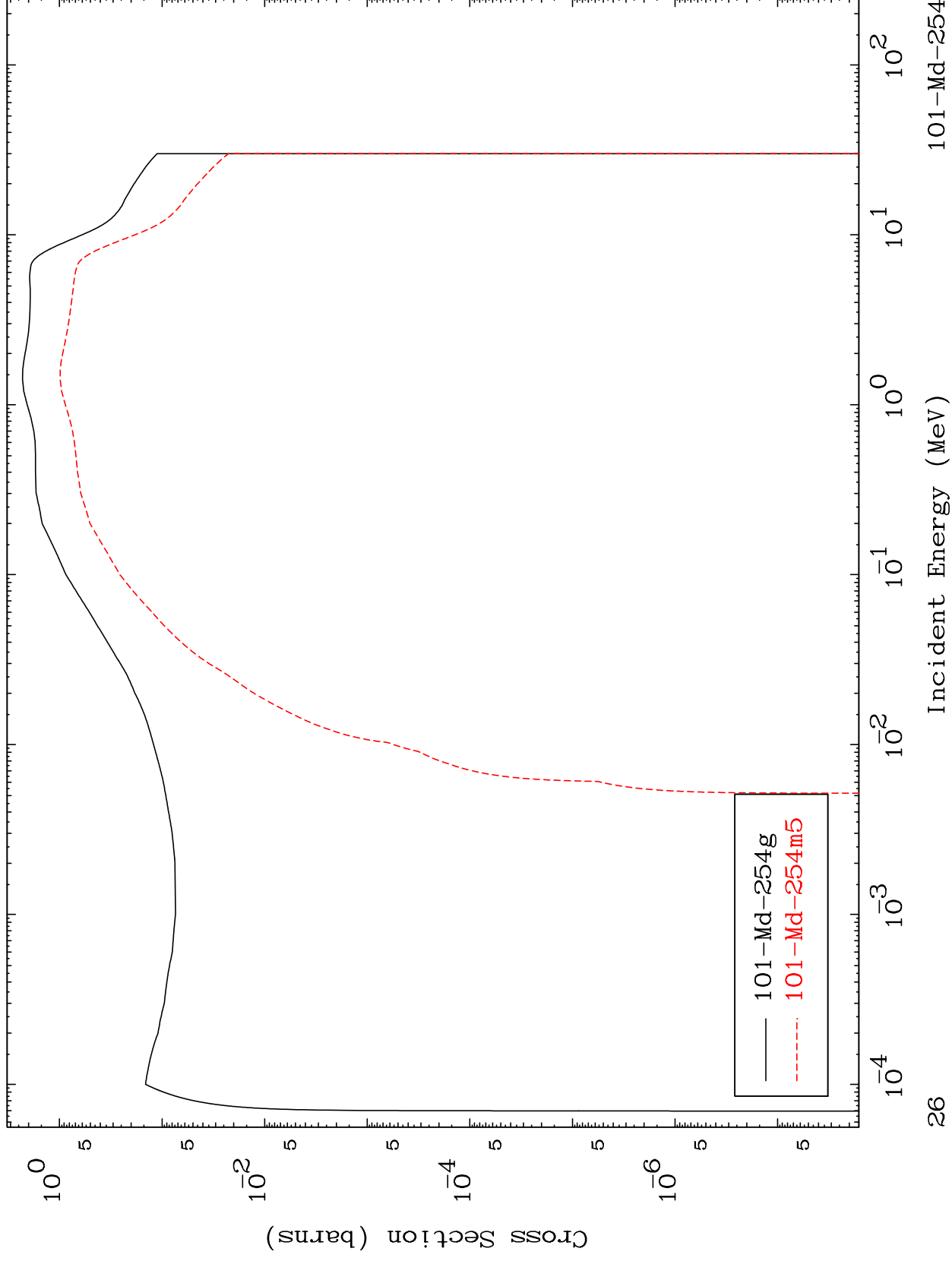
Fission
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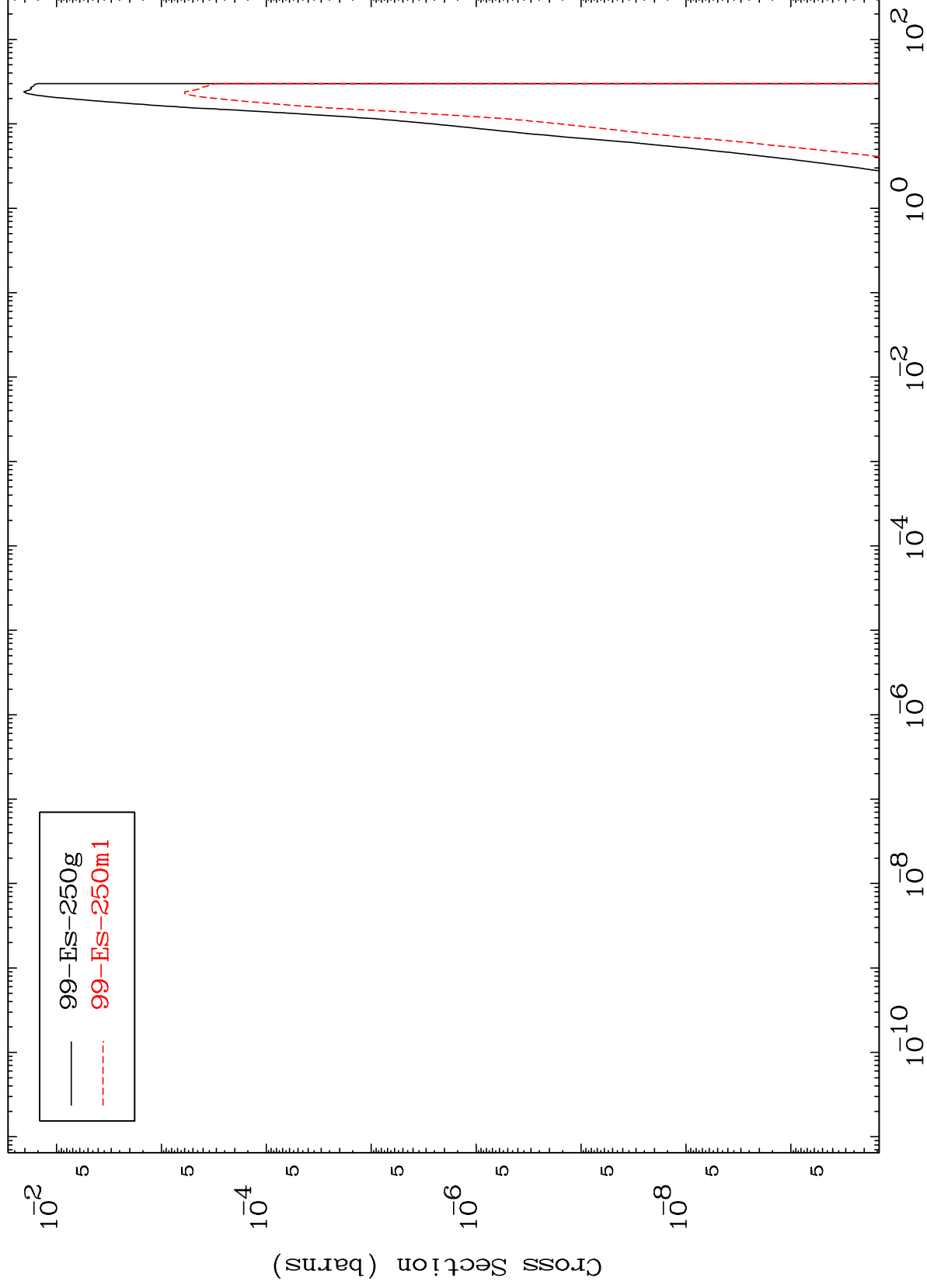


Inelastic
Radionuclide Production Cross Section

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(n,n') α
Radionuclide Production Cross Section



27

101-Md-254