

Program EVALPLOT  
(Version 2017-1)

by

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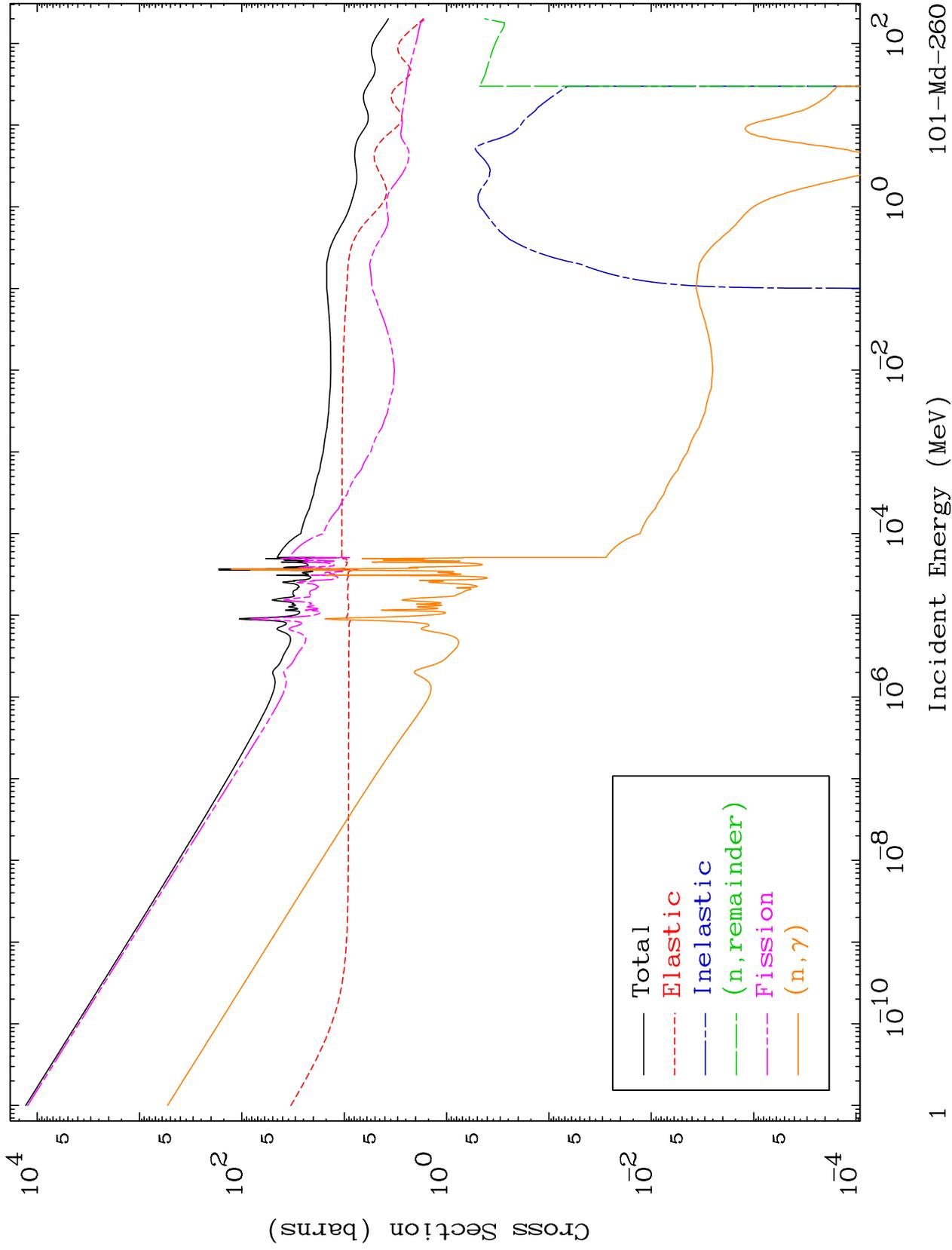
Press Mouse Button to Start

MAT 9961

Major

293 Kelvin Cross Sections

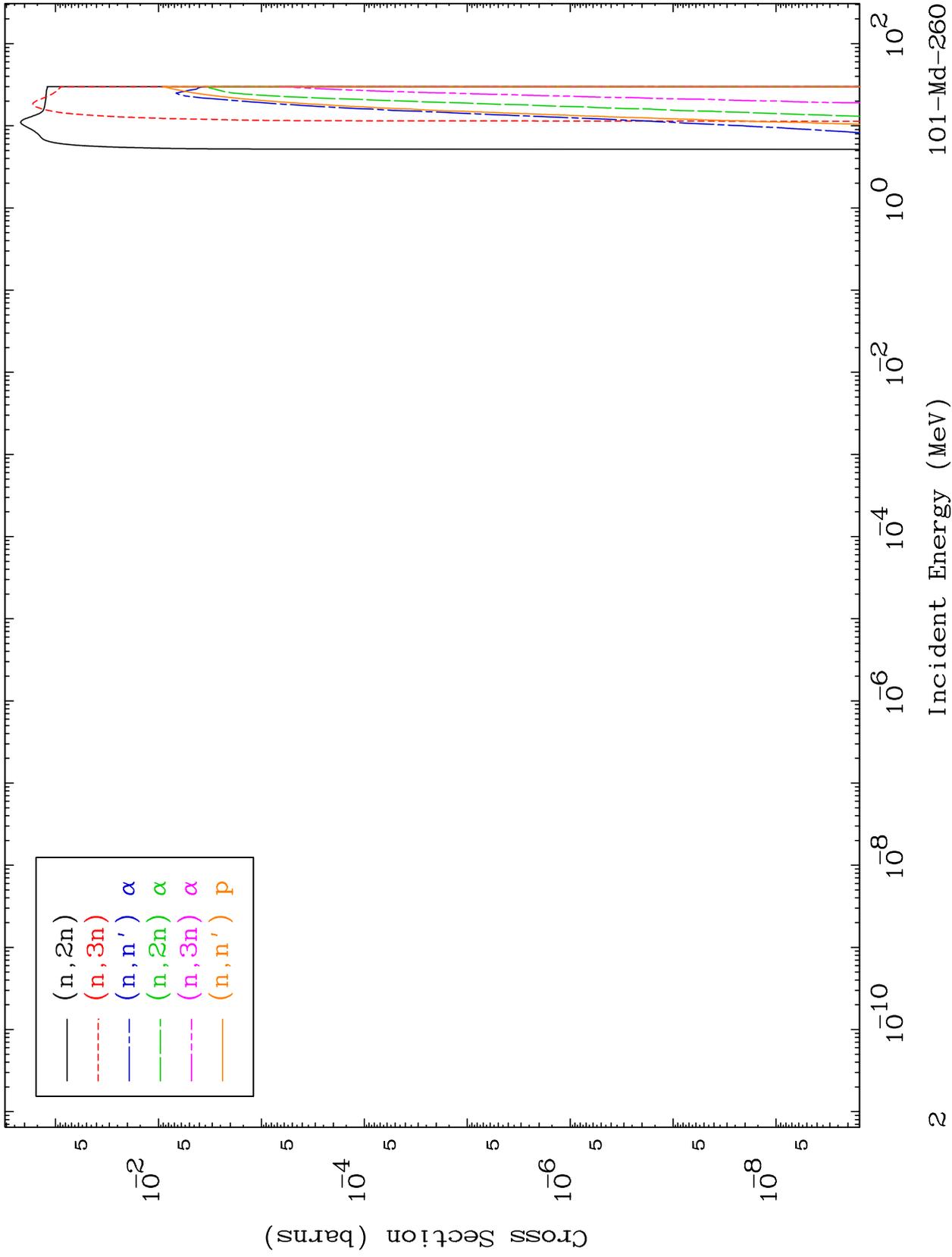
101-Md-260

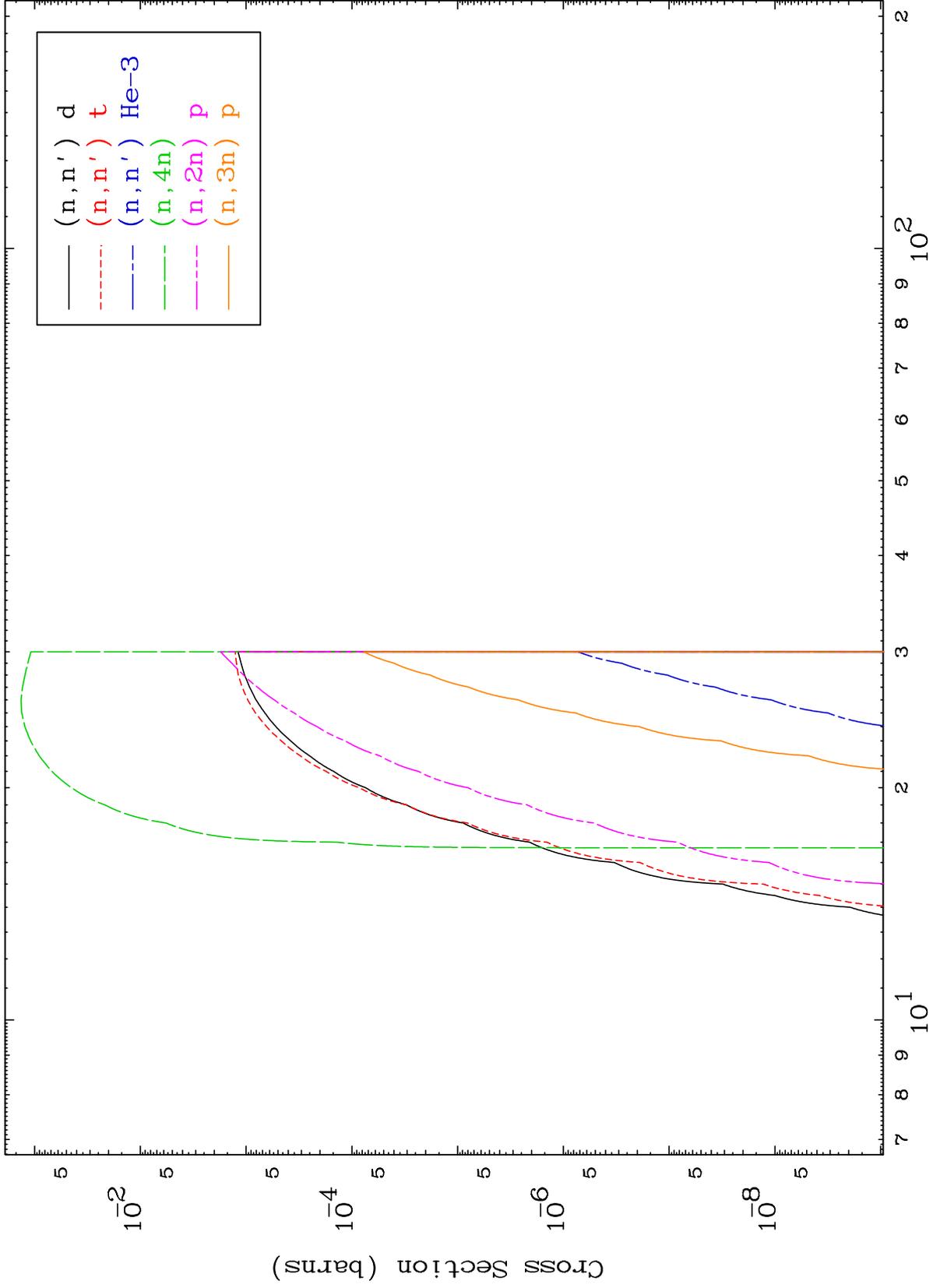


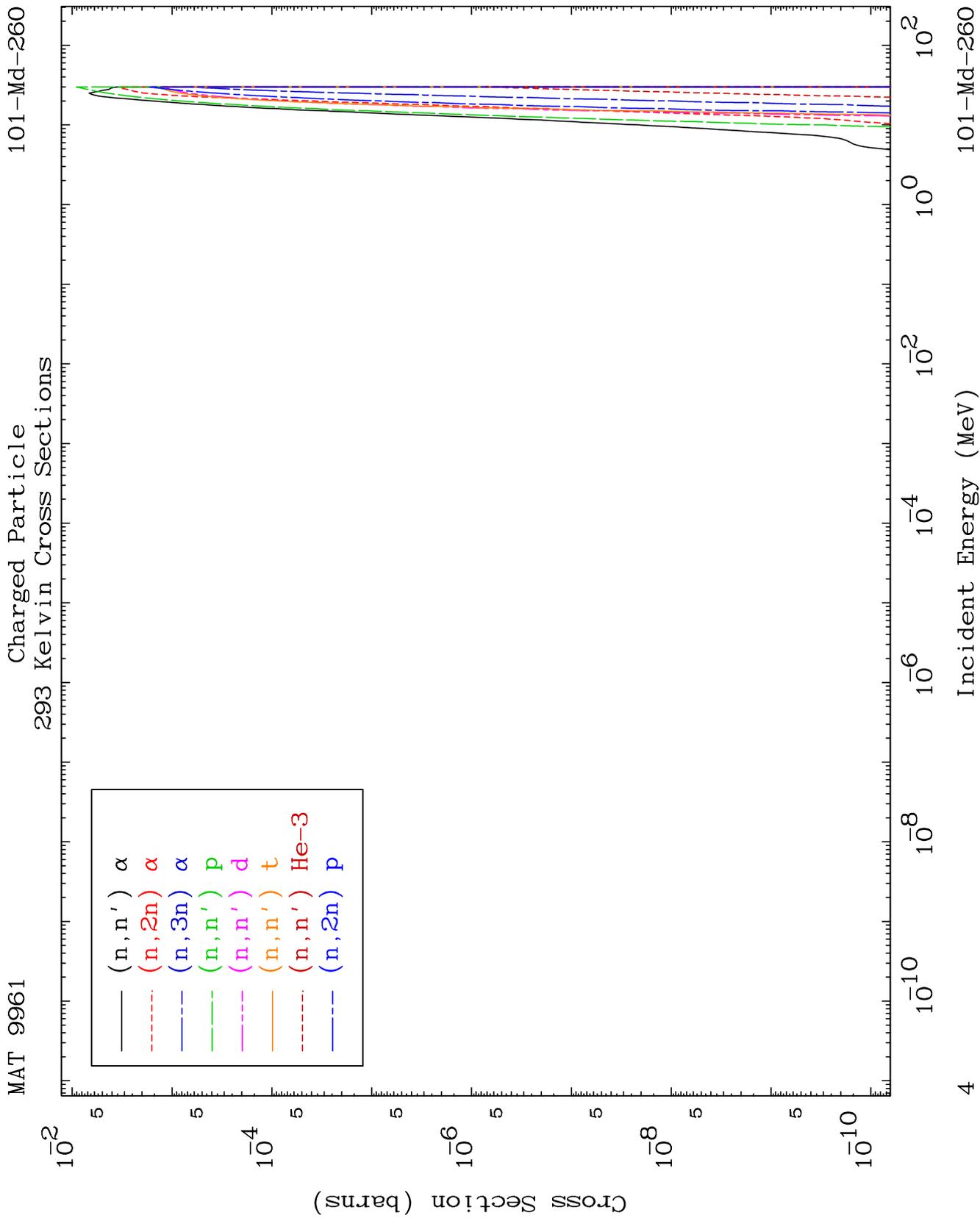
MAT 9961

Neutron Production  
293 Kelvin Cross Sections

101-Md-260



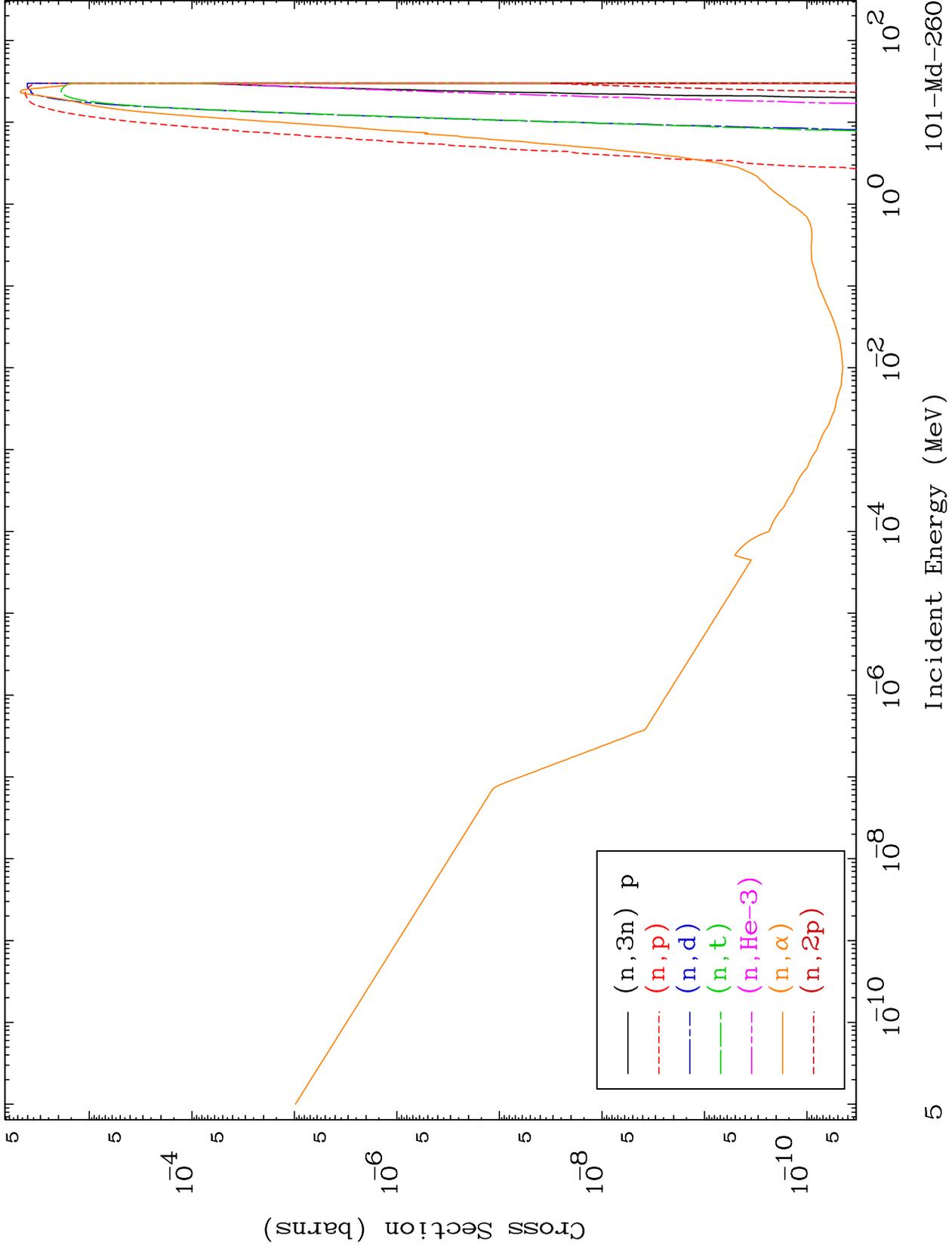


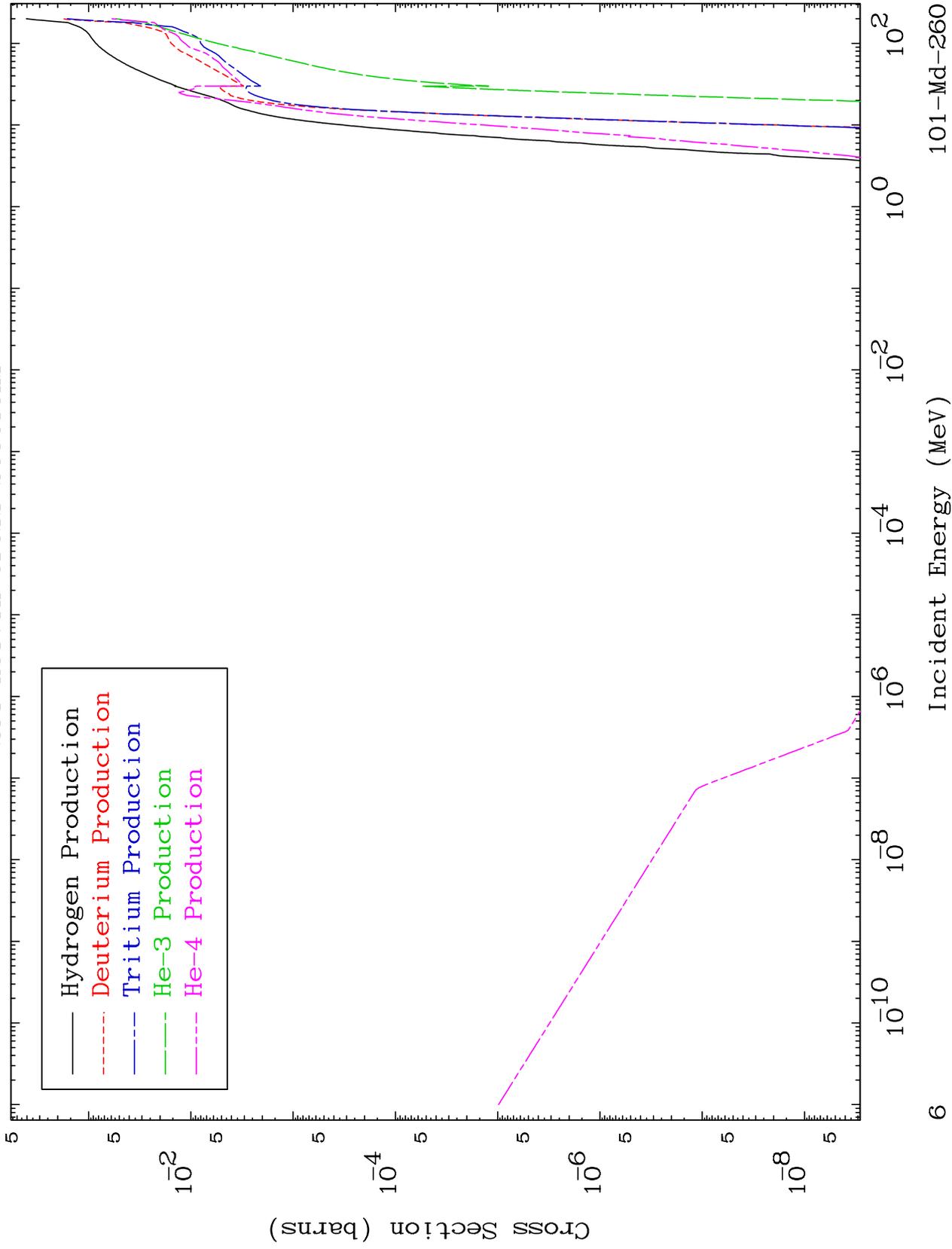


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Charged Particle  
293 Kelvin Cross Sections

101-Md-260

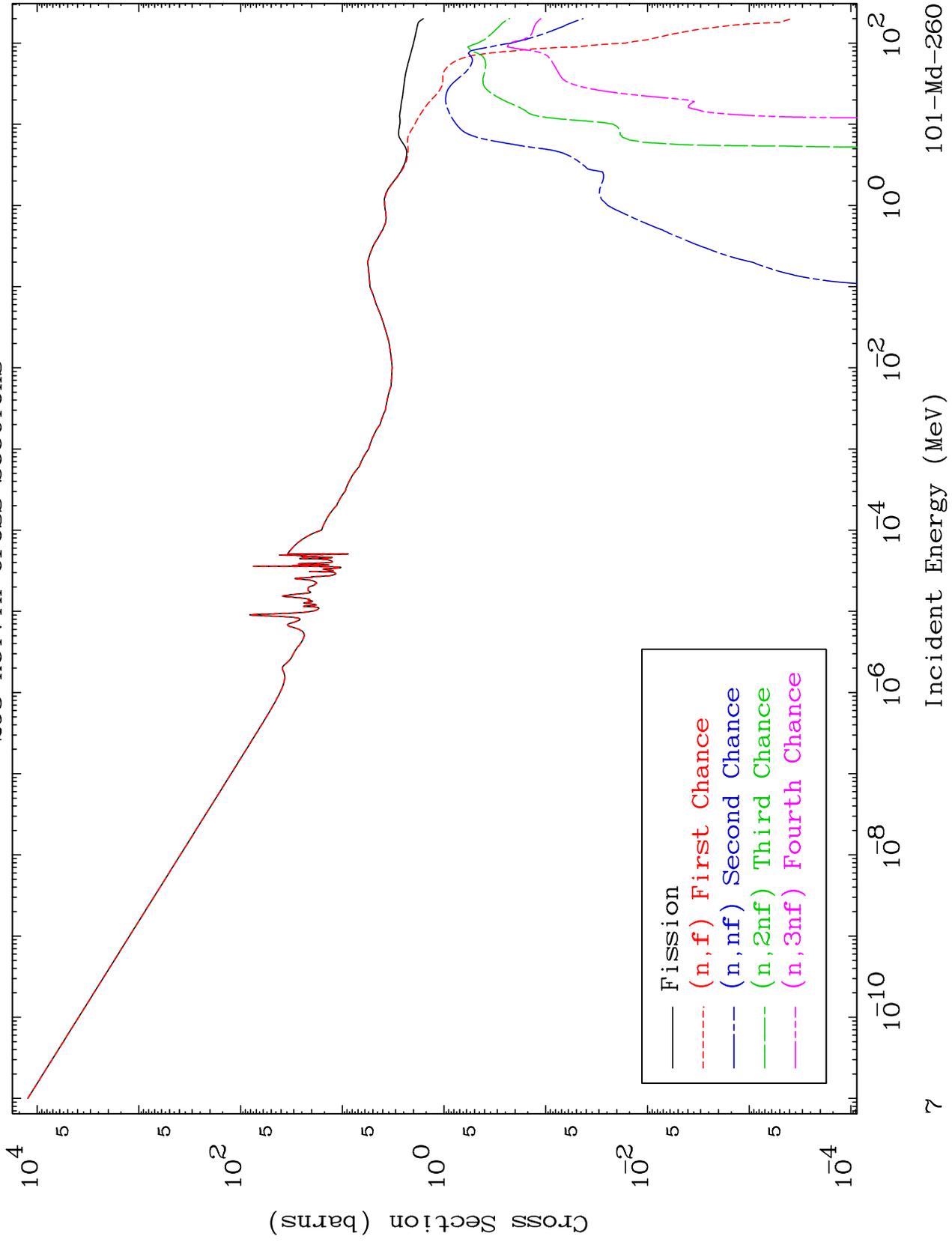




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Fission  
293 Kelvin Cross Sections

101-Md-260

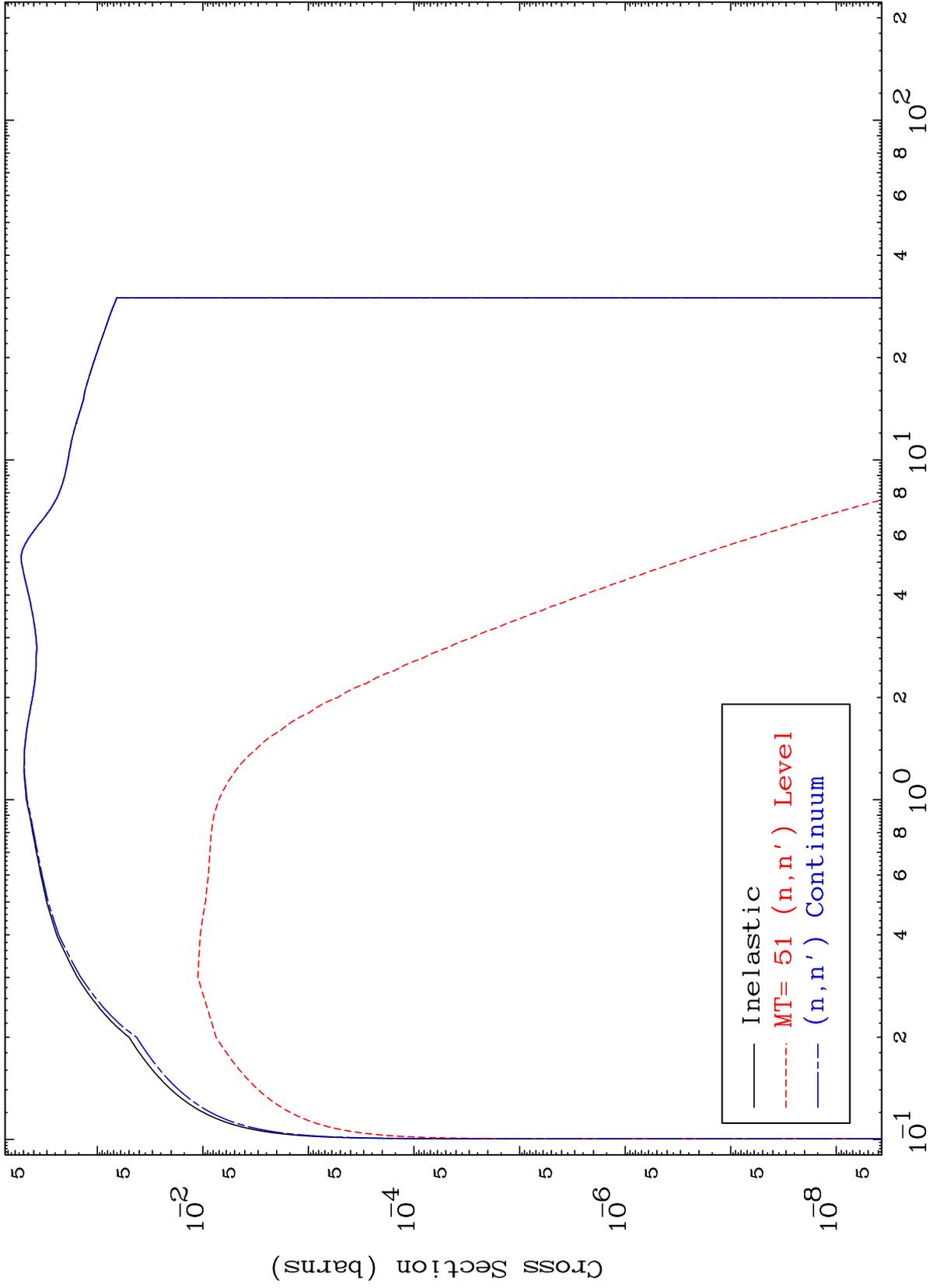


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(n,n') Level

101-Md-260

293 Kelvin Cross Sections



Incident Energy (MeV)

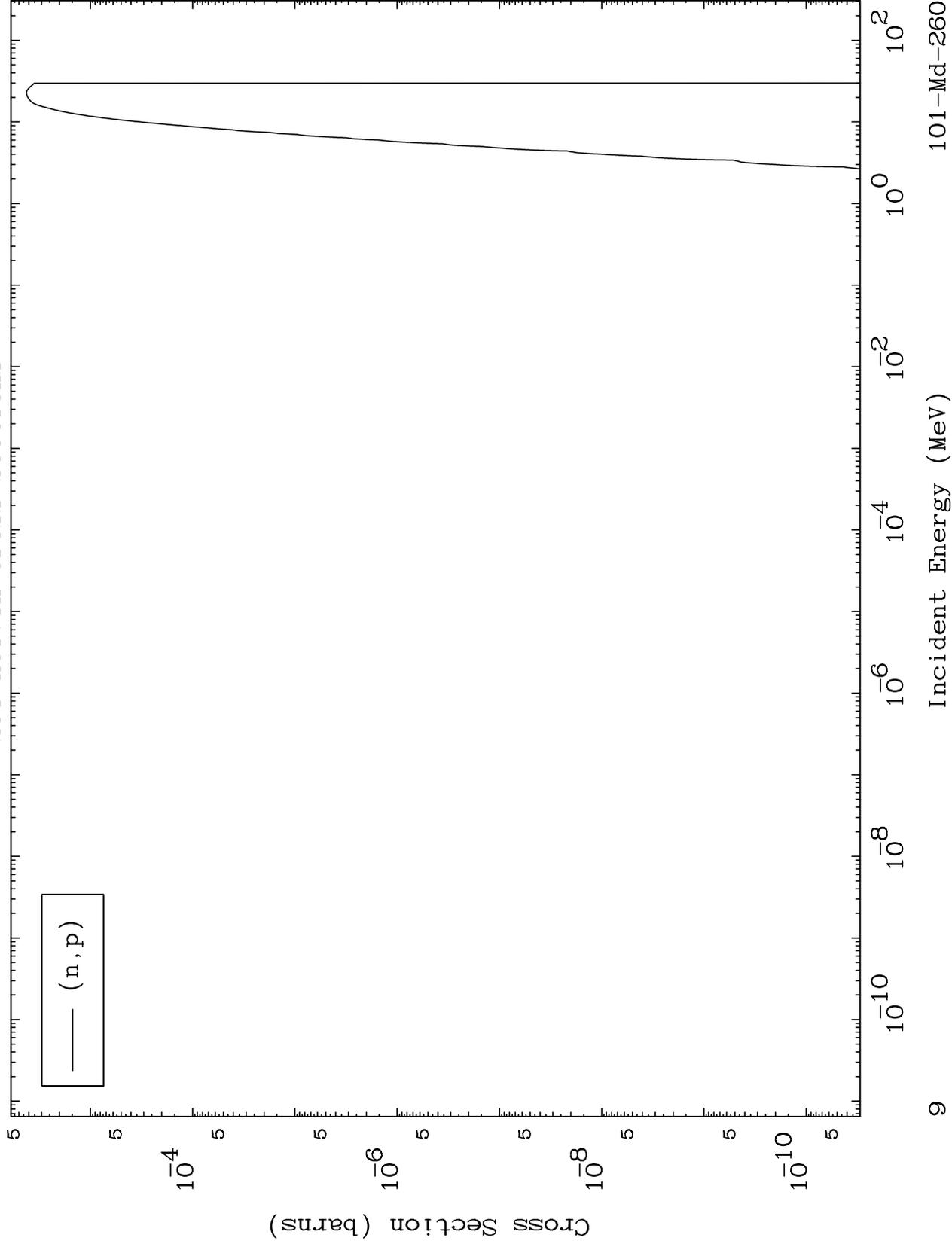
101-Md-260

8

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(n,p) Levels  
293 Kelvin Cross Sections

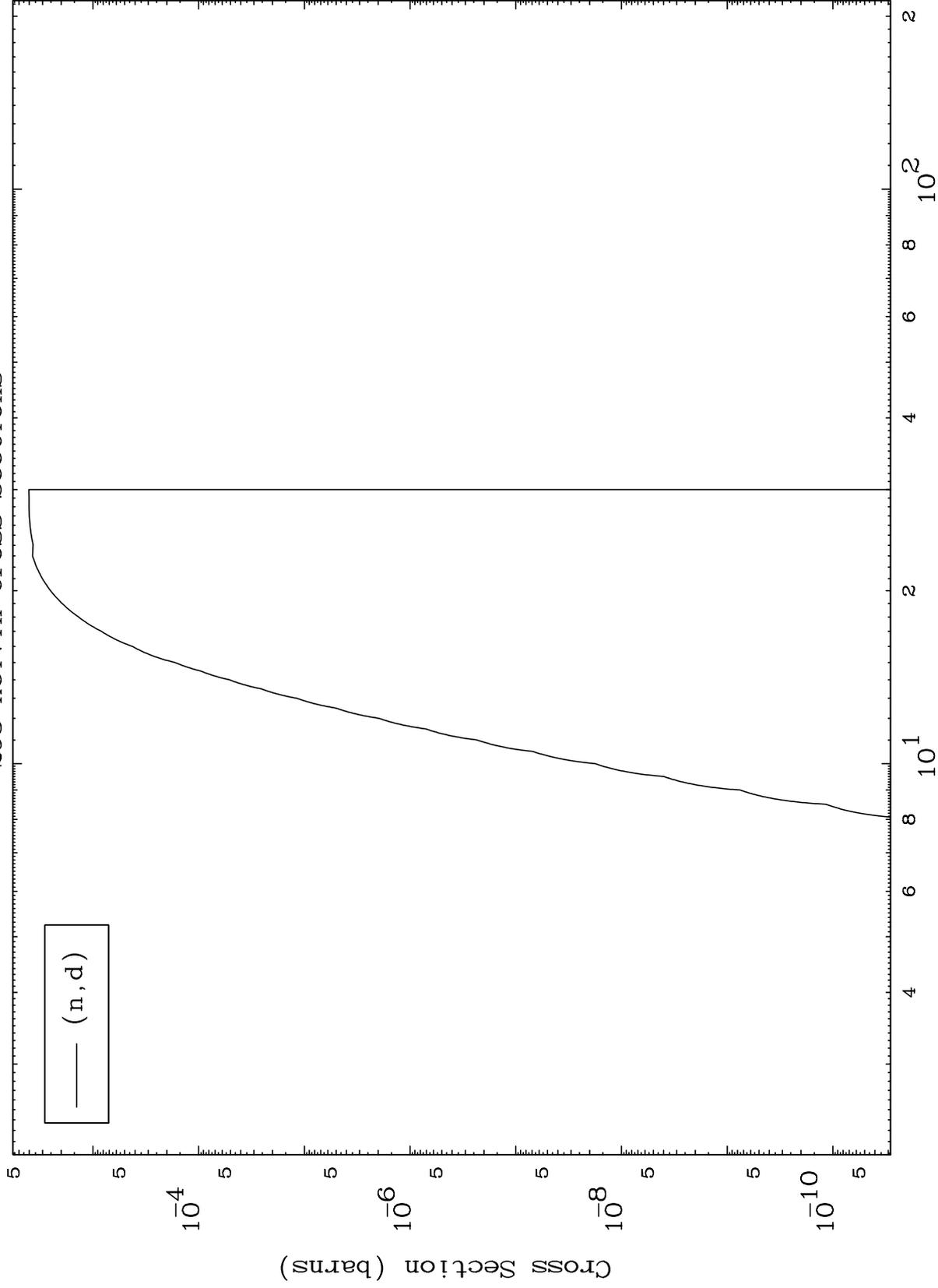
101-Md-260



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(n,d) Levels  
293 Kelvin Cross Sections

101-Md-260



10

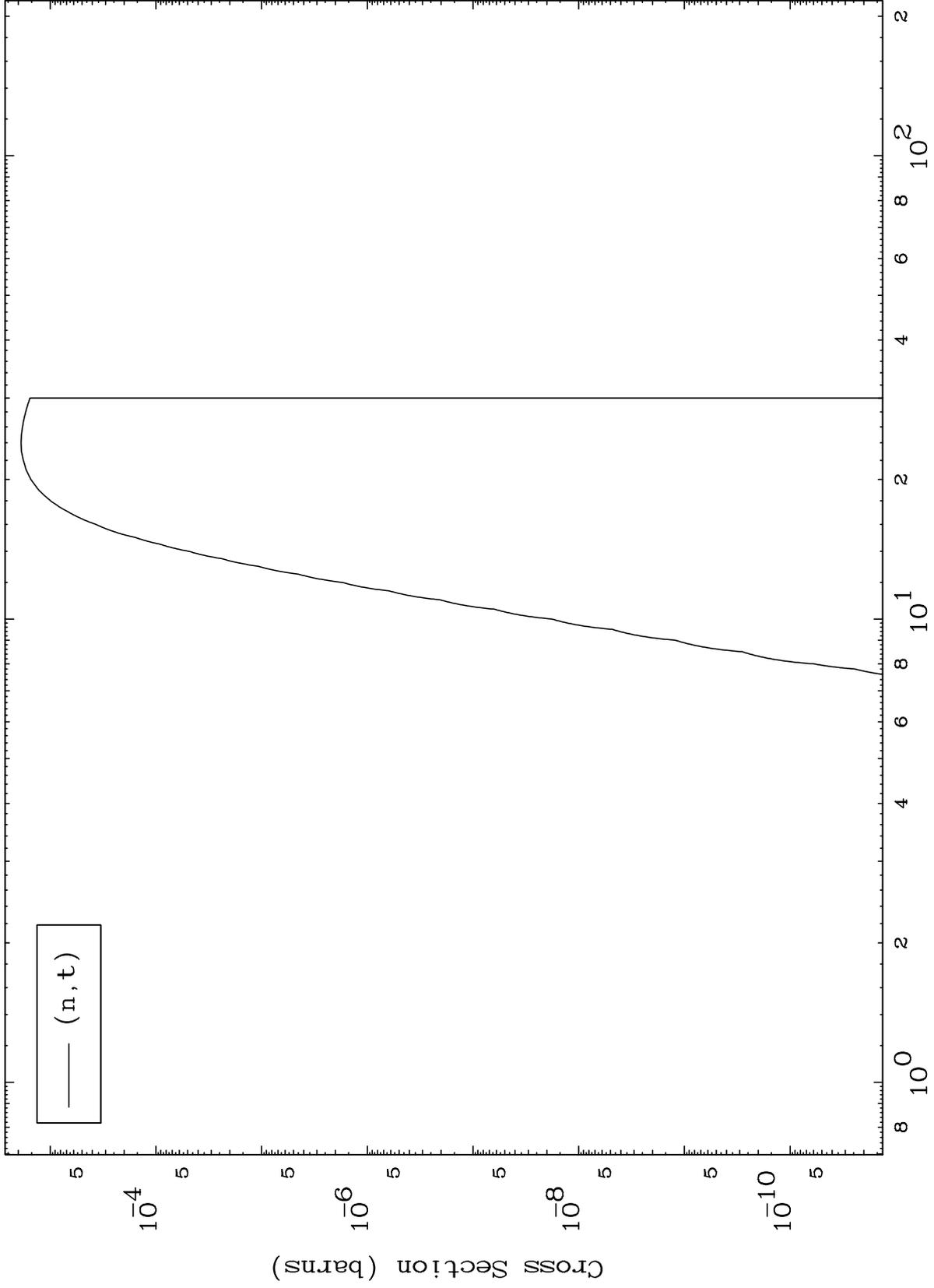
Incident Energy (MeV)

101-Md-260

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(n,t) Levels  
293 Kelvin Cross Sections

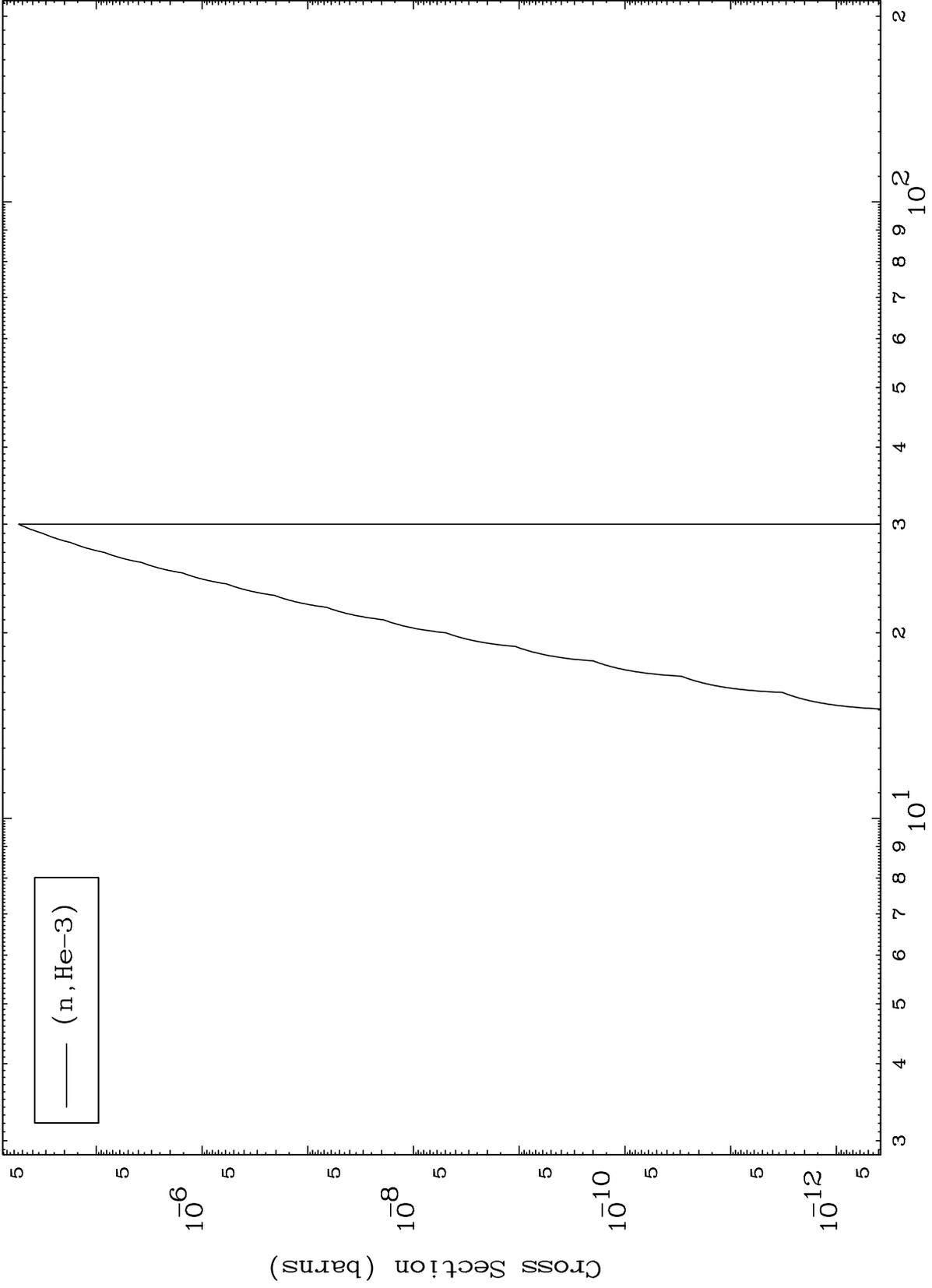
101-Md-260



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(n,He3) Levels  
293 Kelvin Cross Sections

101-Md-260



12

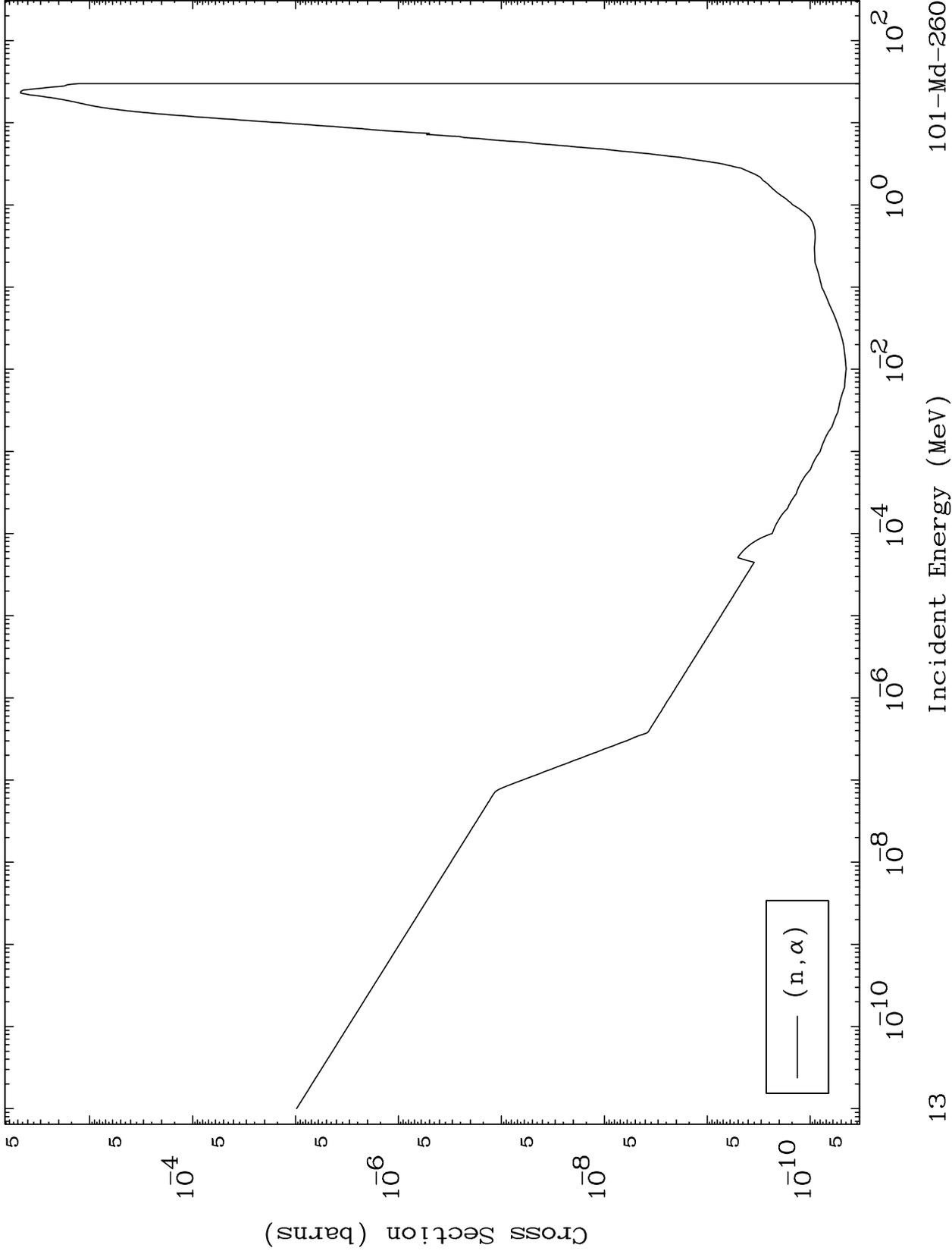
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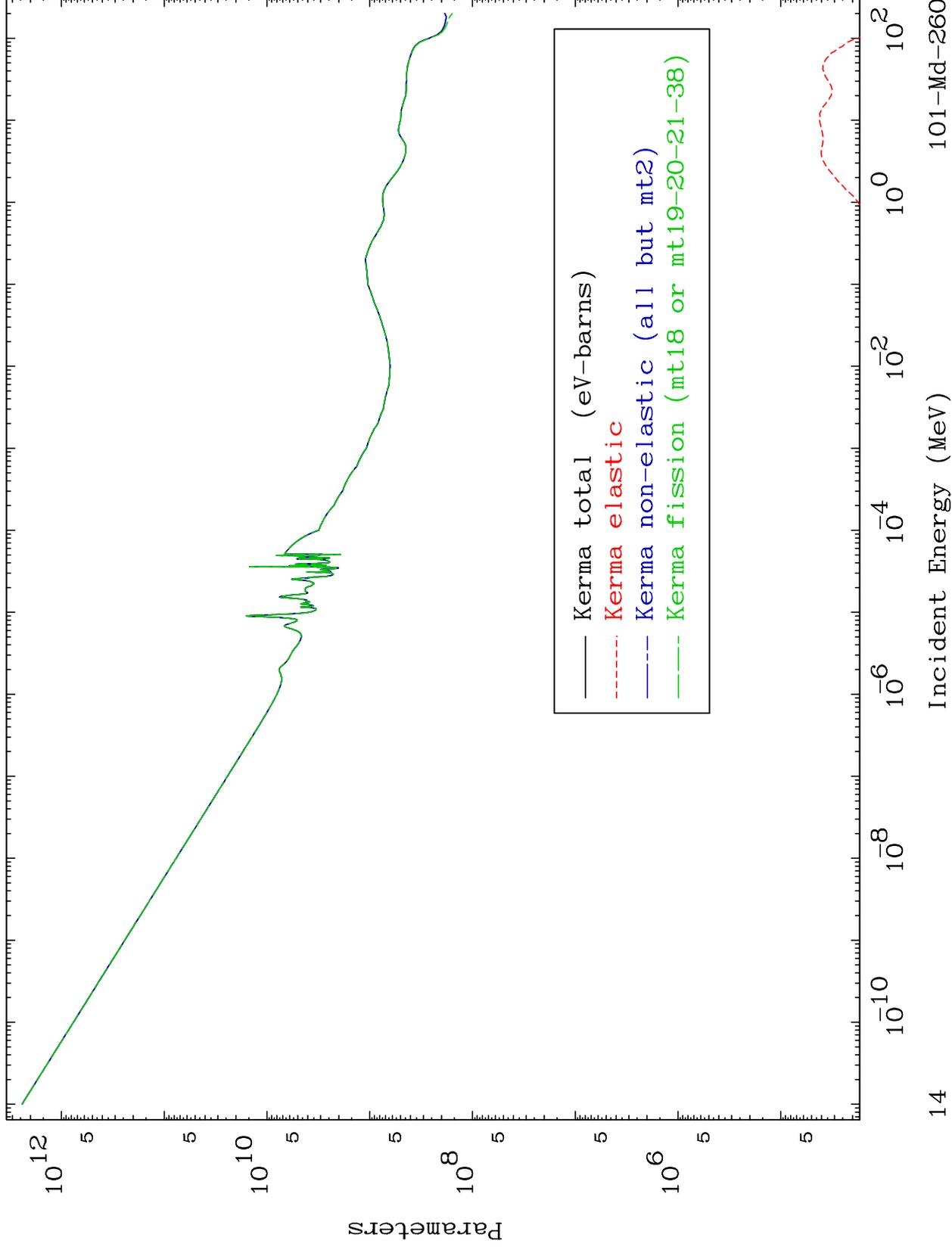
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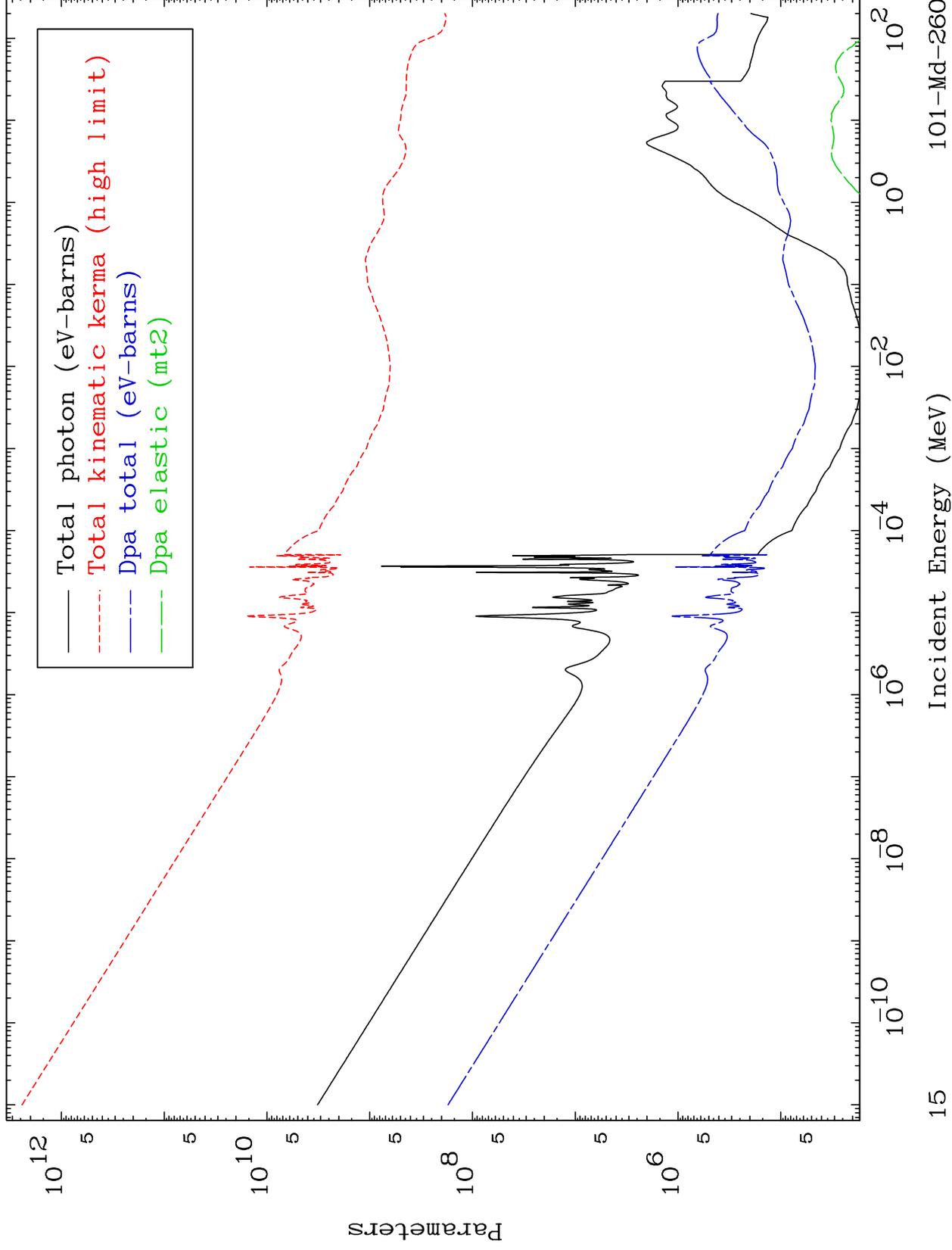
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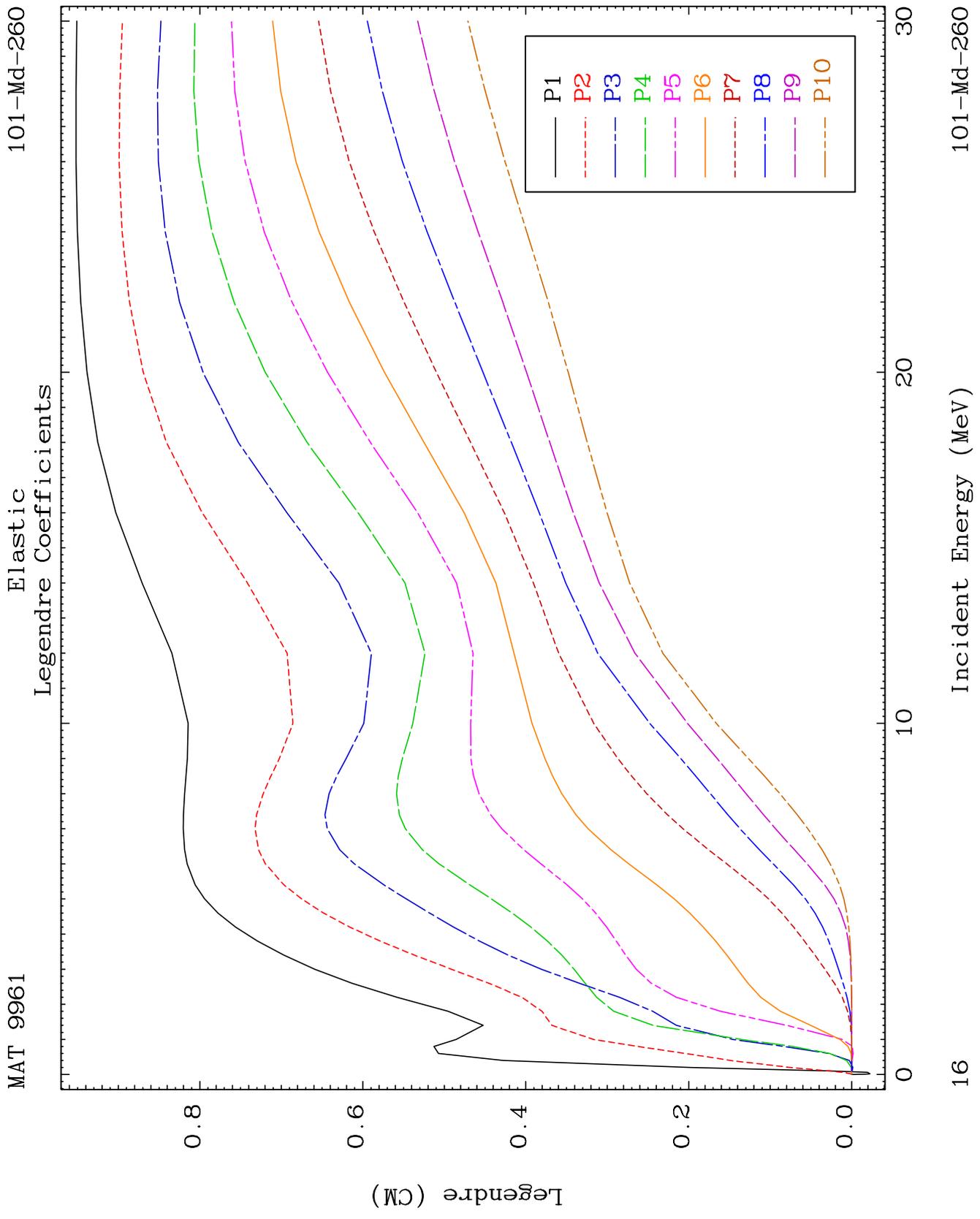
(n,α) Levels  
293 Kelvin Cross Sections

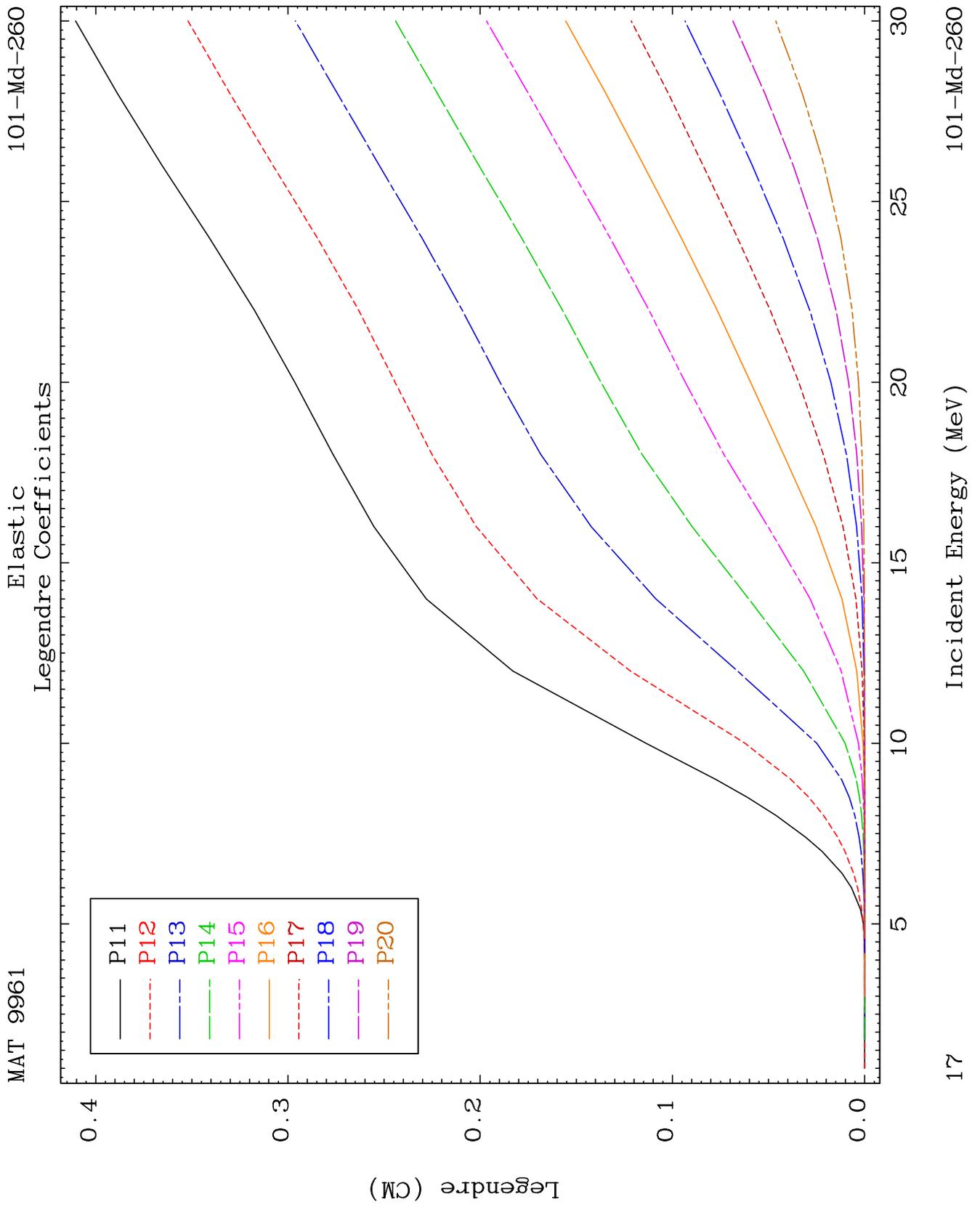
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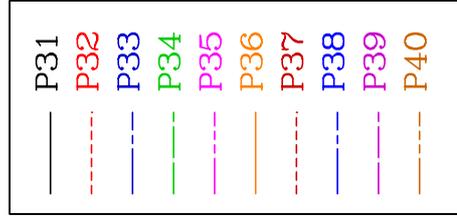




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Elastic  
Legendre Coefficients

101-Md-260



$\times 10^{-6}$

Legendre (CM)

3

2

1

0

15

20

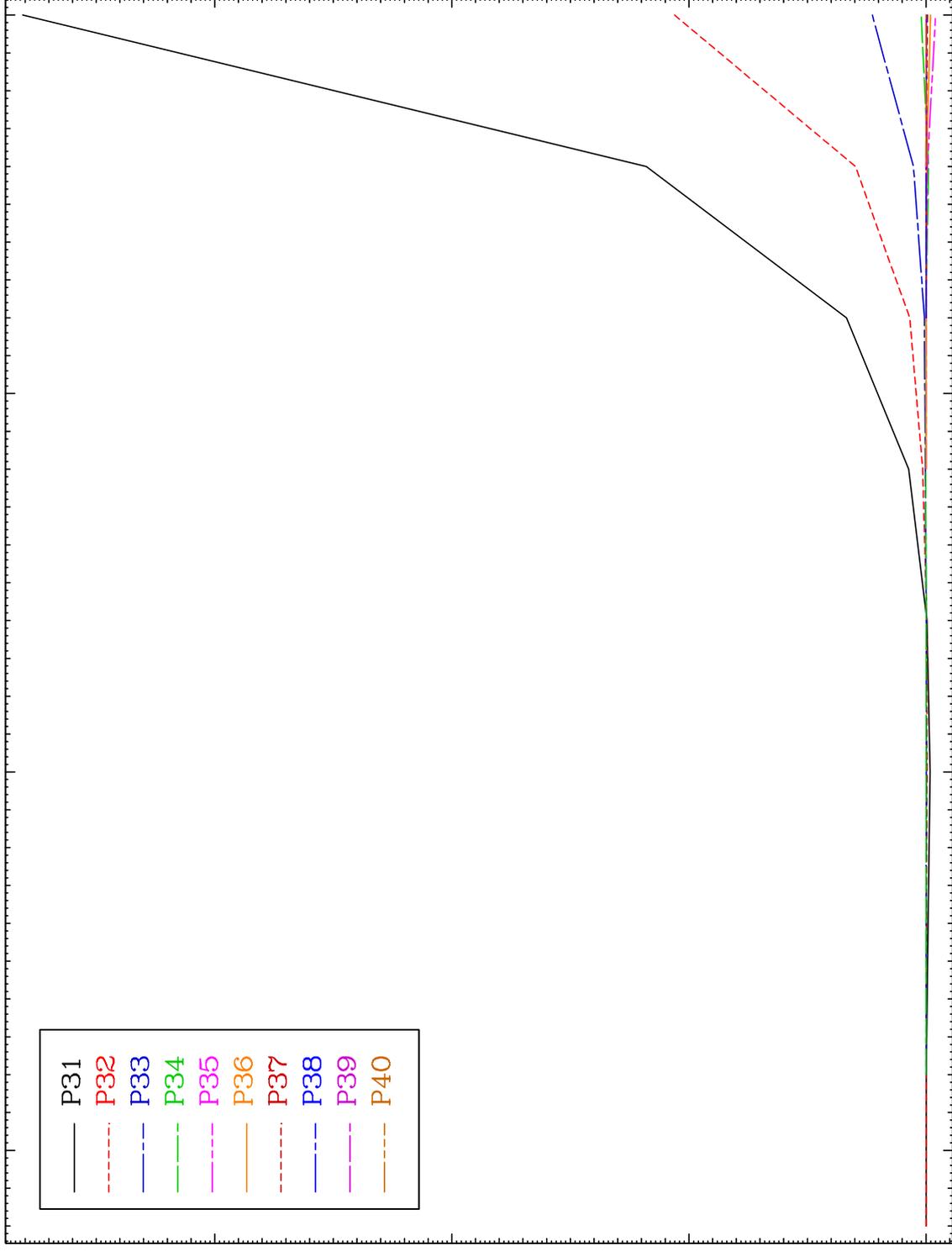
25

30

19

Incident Energy (MeV)

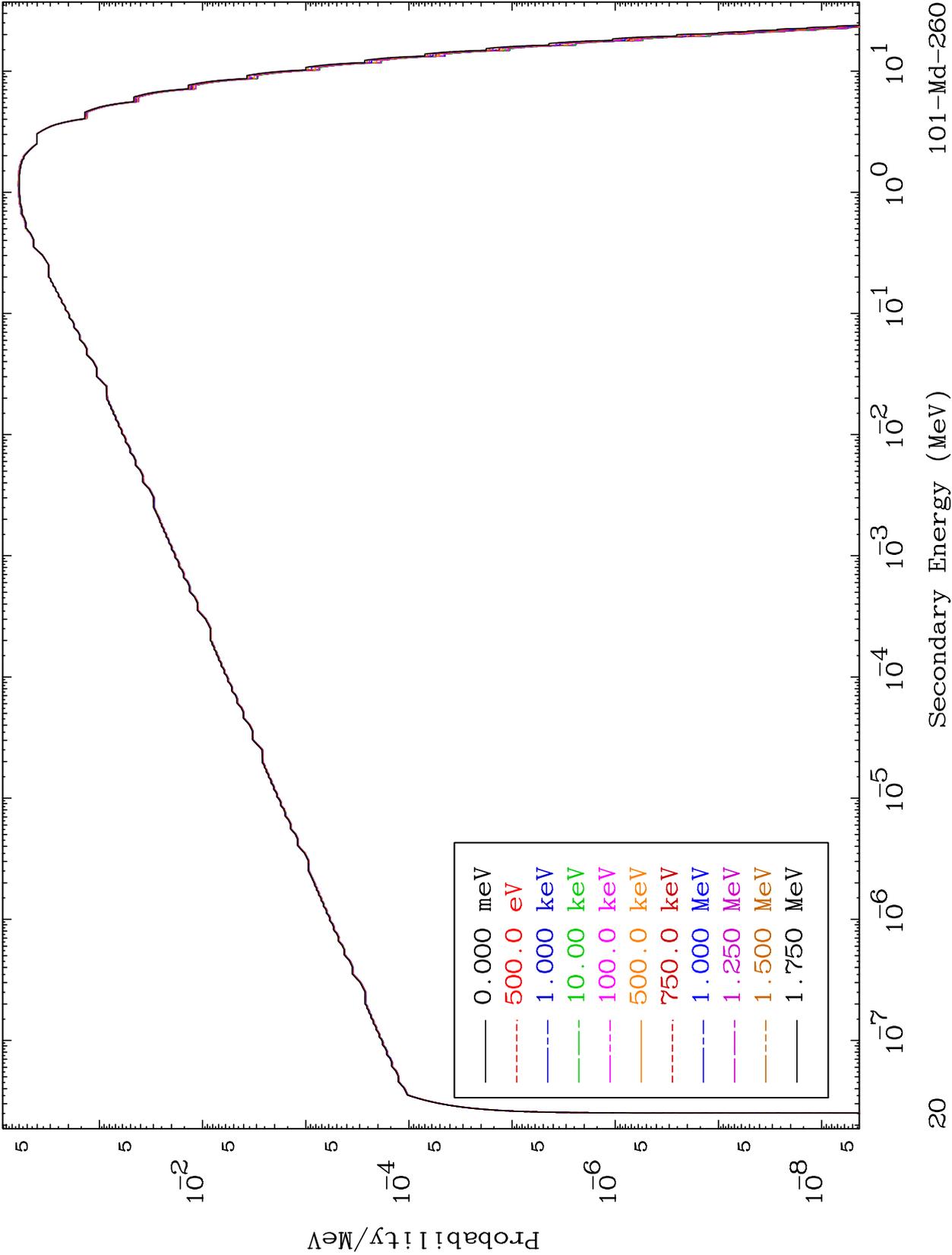
101-Md-260



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### Fission Energy Distribution

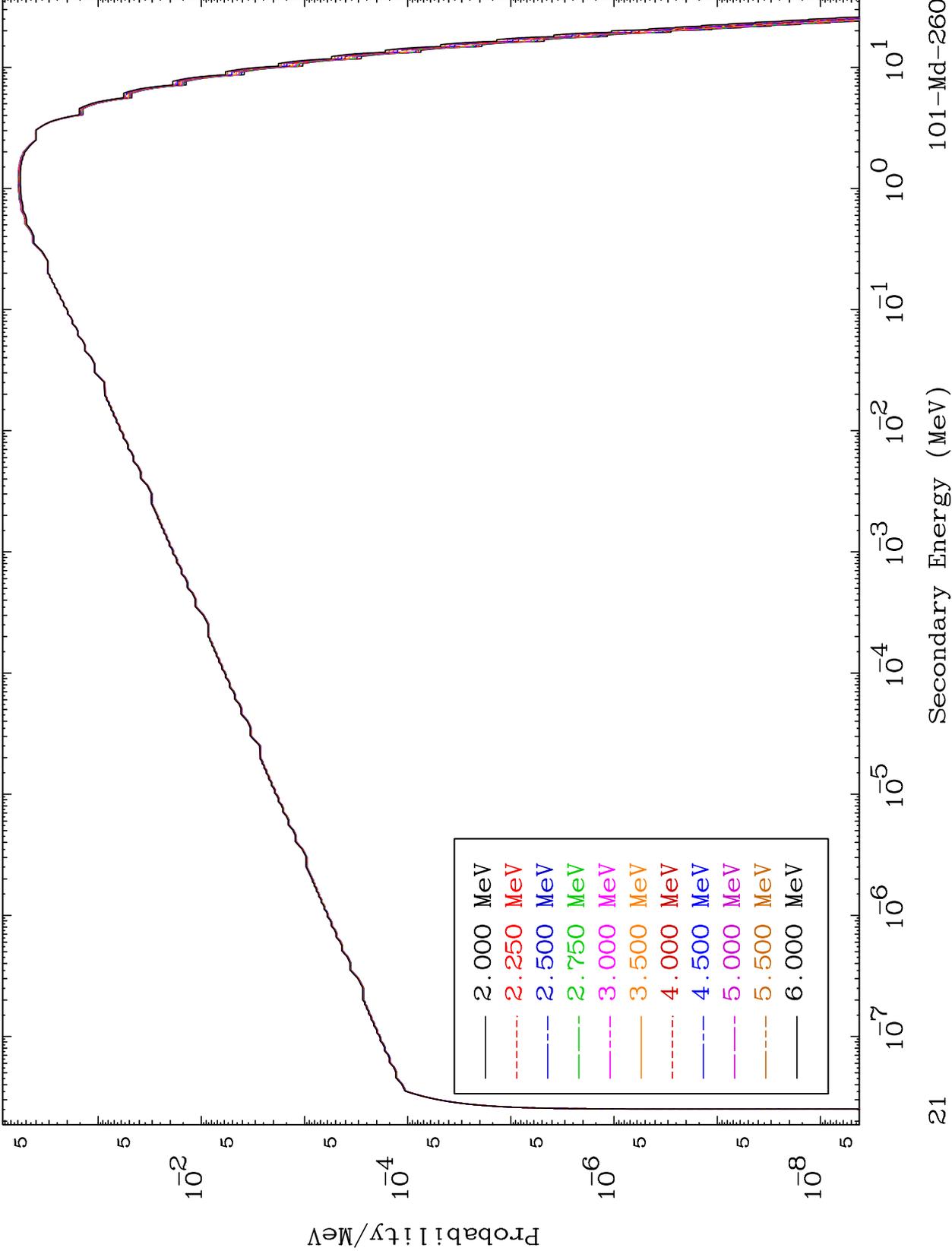
101-Md-260



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### Fission Energy Distribution

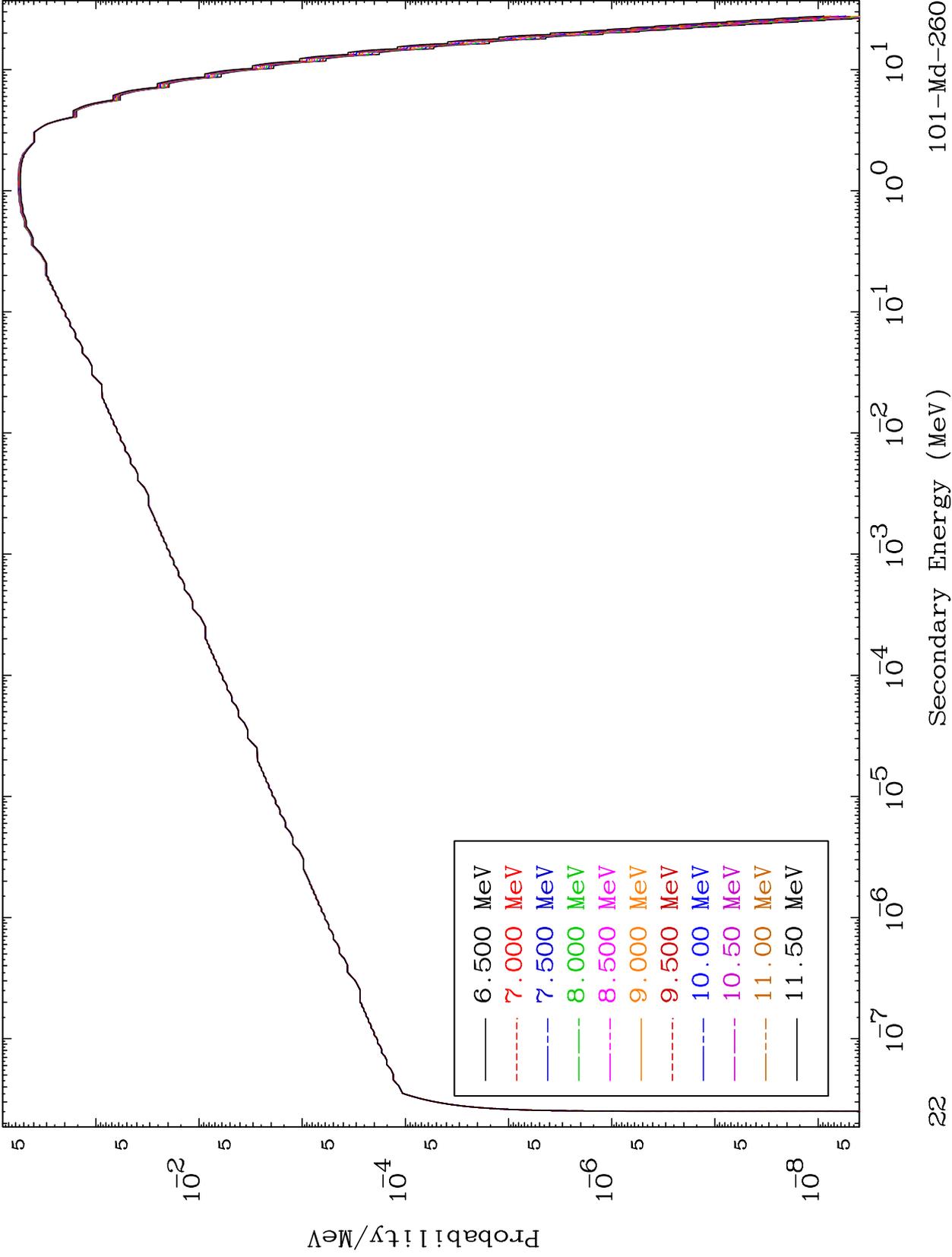
101-Md-260



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# Fission Energy Distribution

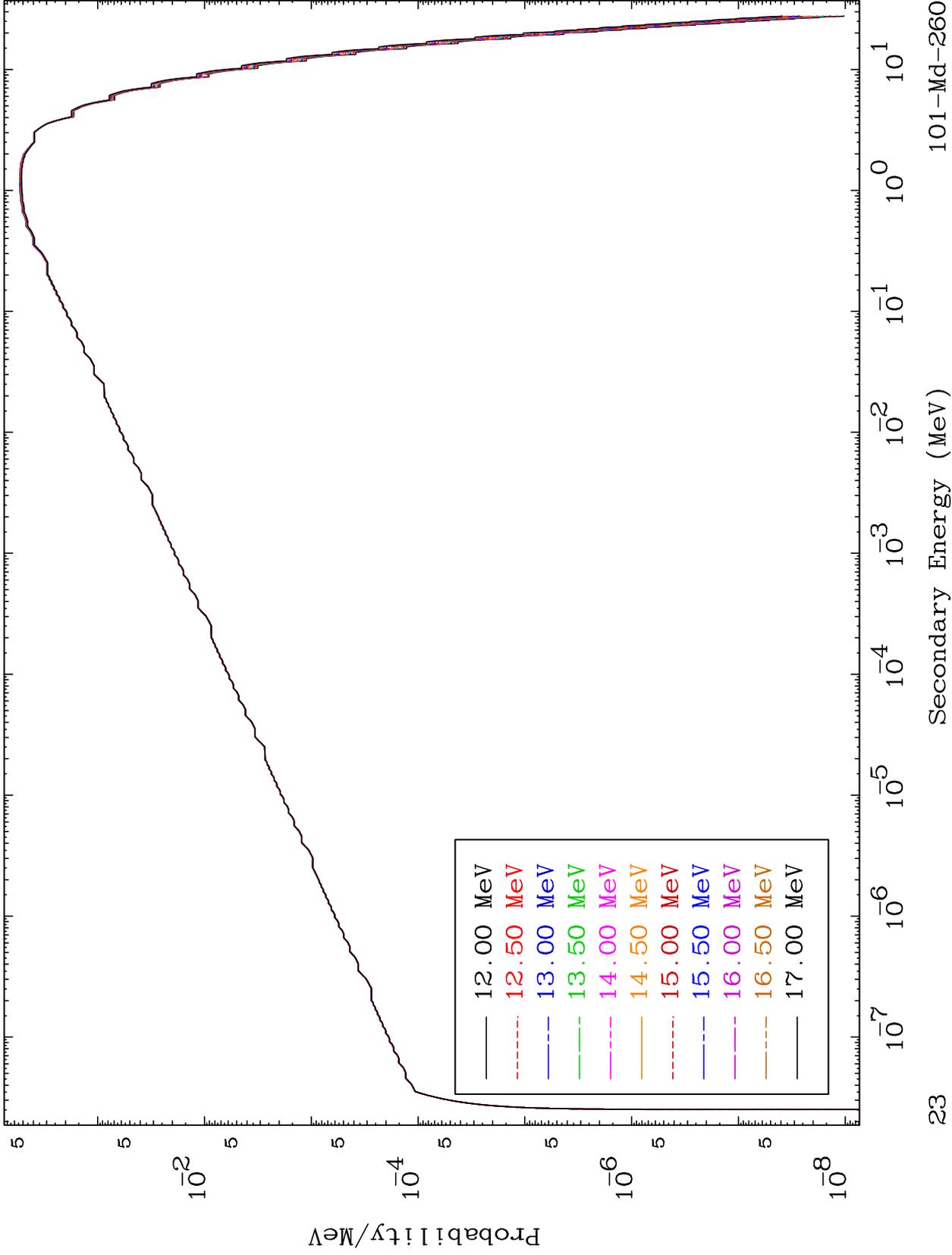
101-Md-260



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Fission  
Energy Distribution

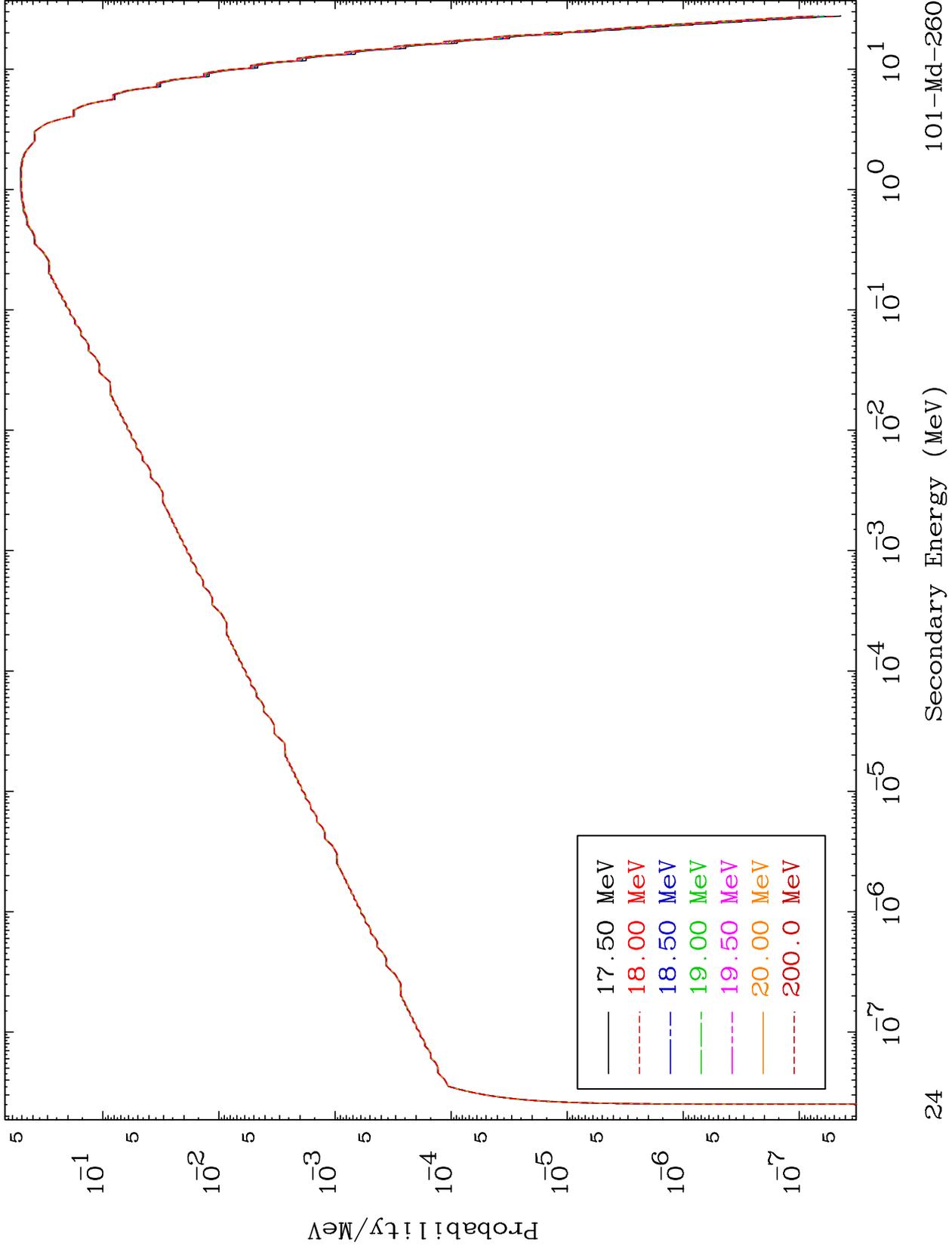
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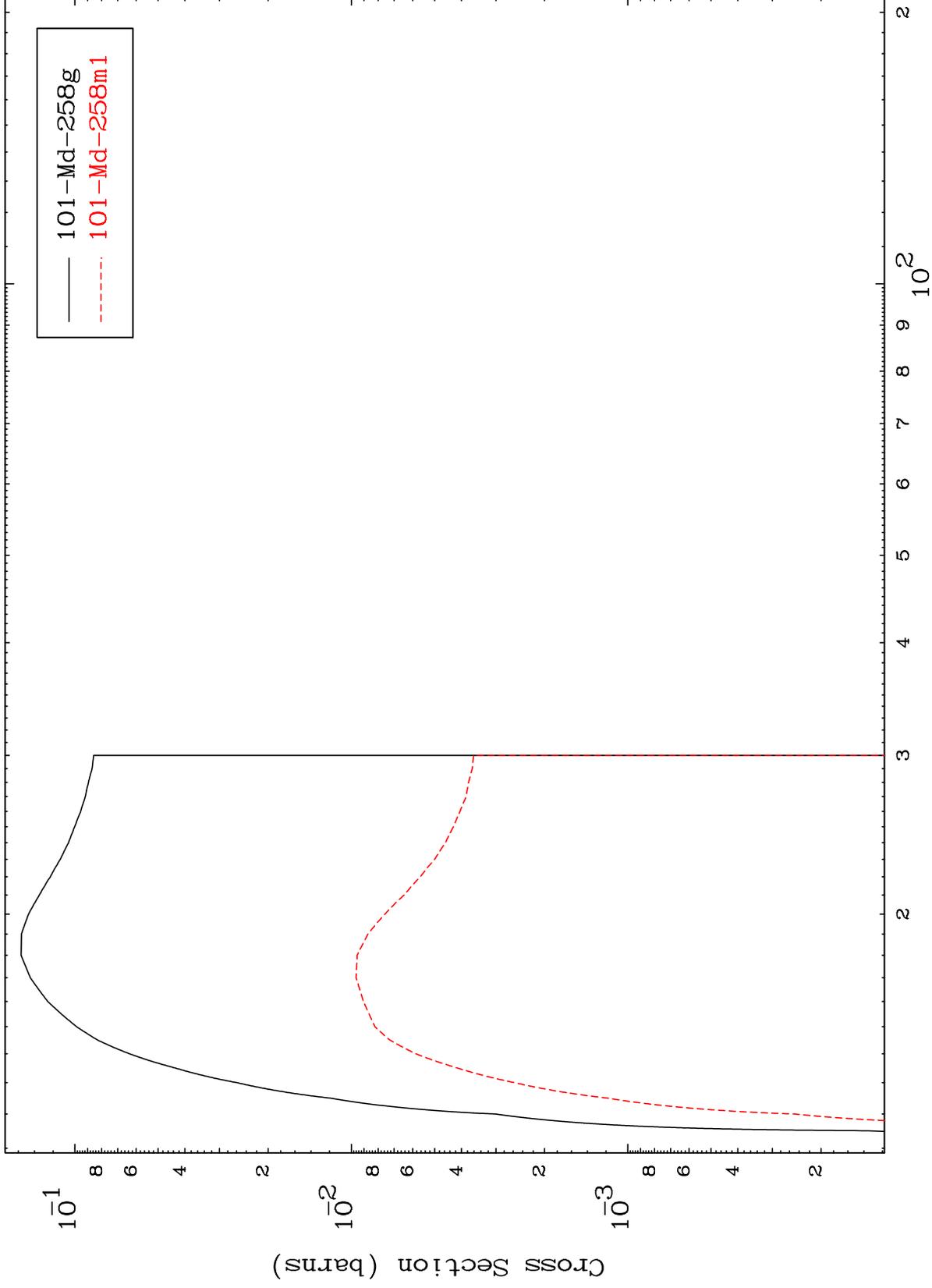
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### Fission Energy Distribution

101-Md-260



Radionuclide Production Cross Section

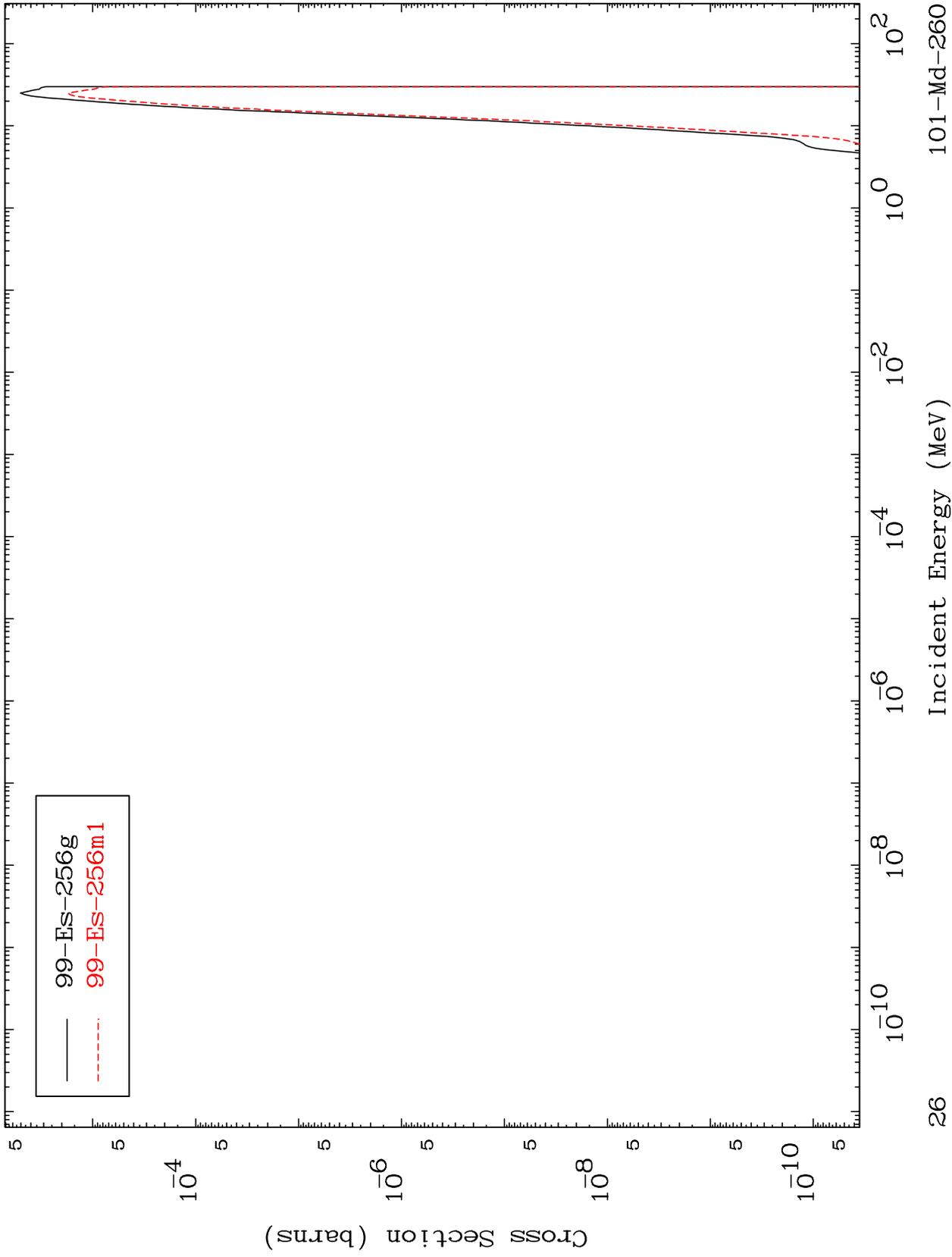


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$(n, n') \alpha$

101-Md-260

Radionuclide Production Cross Section

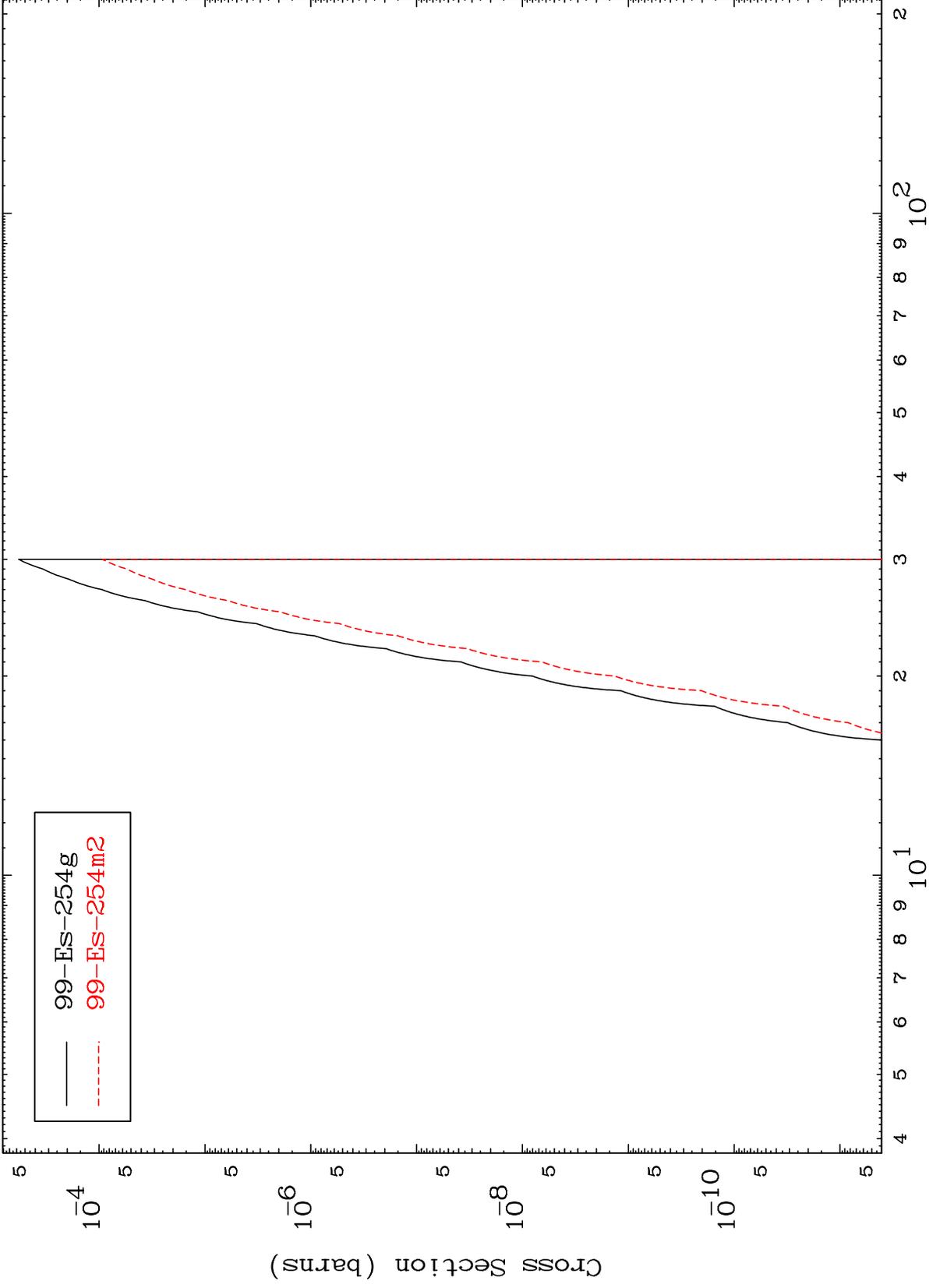


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(n,3n)  $\alpha$

101-Md-260

Radionuclide Production Cross Section



27

Incident Energy (MeV)

101-Md-260