

Program EVALPLOT
(Version 2017-1)

by

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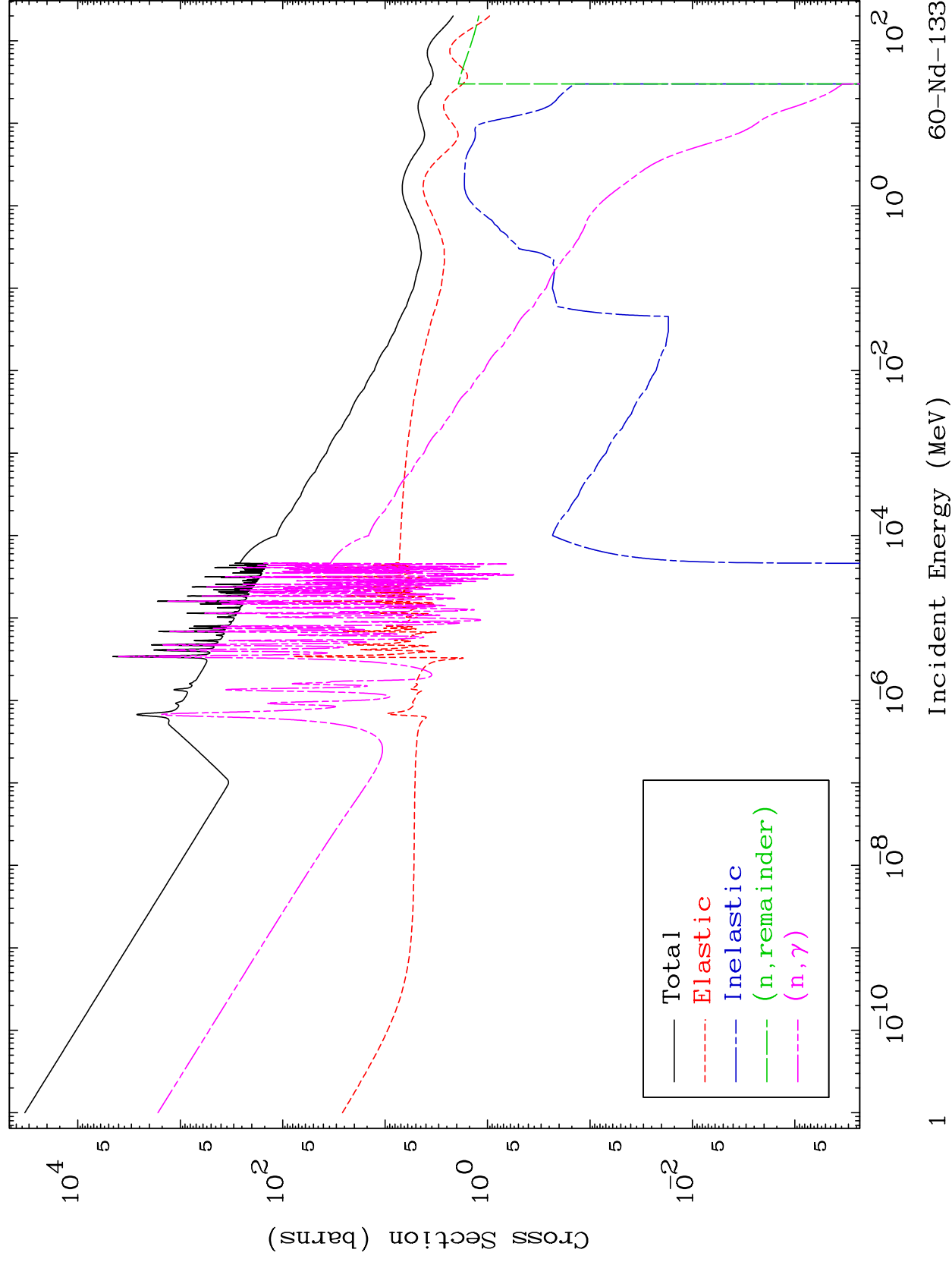
Press Mouse Button to Start

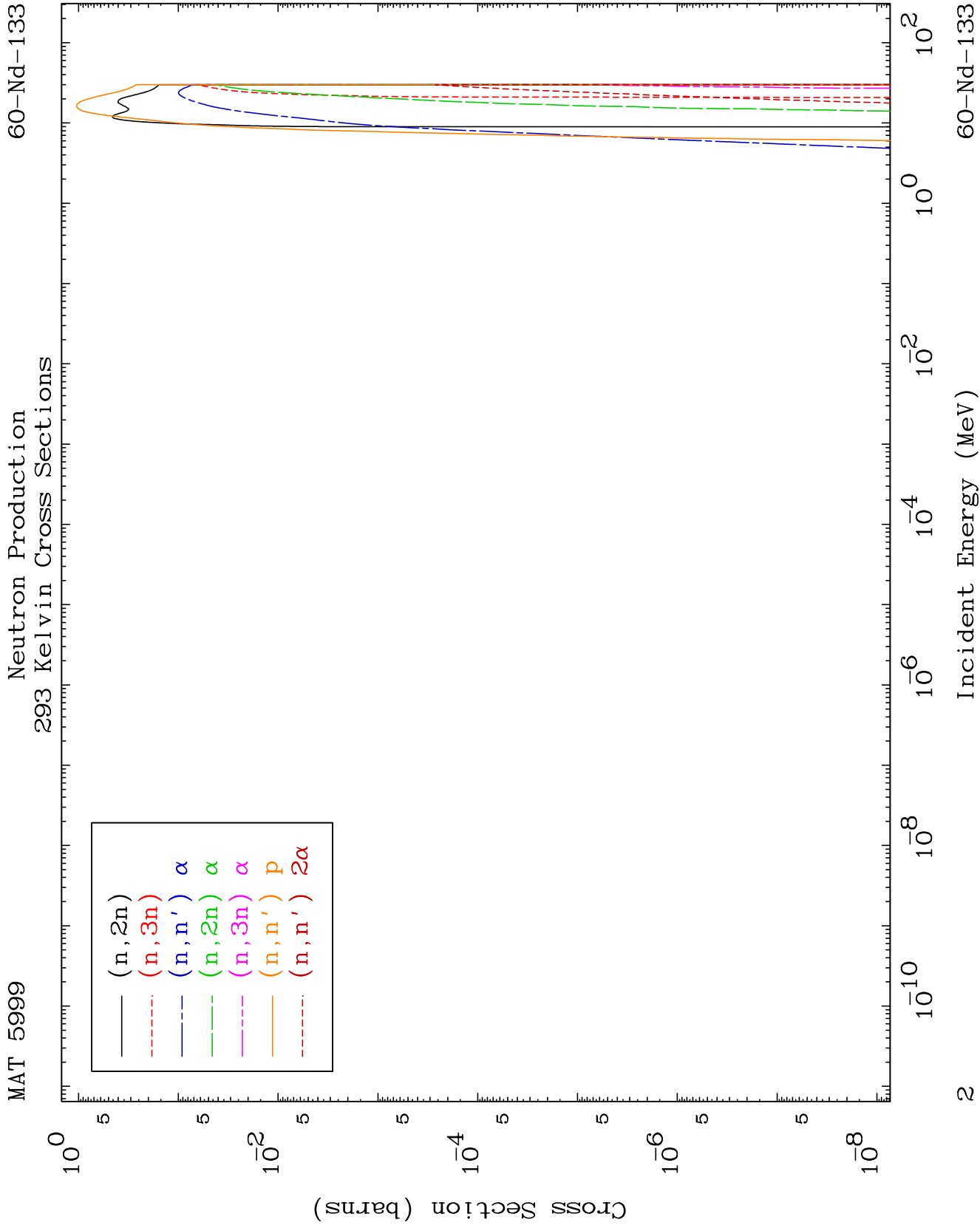
MAT 5999

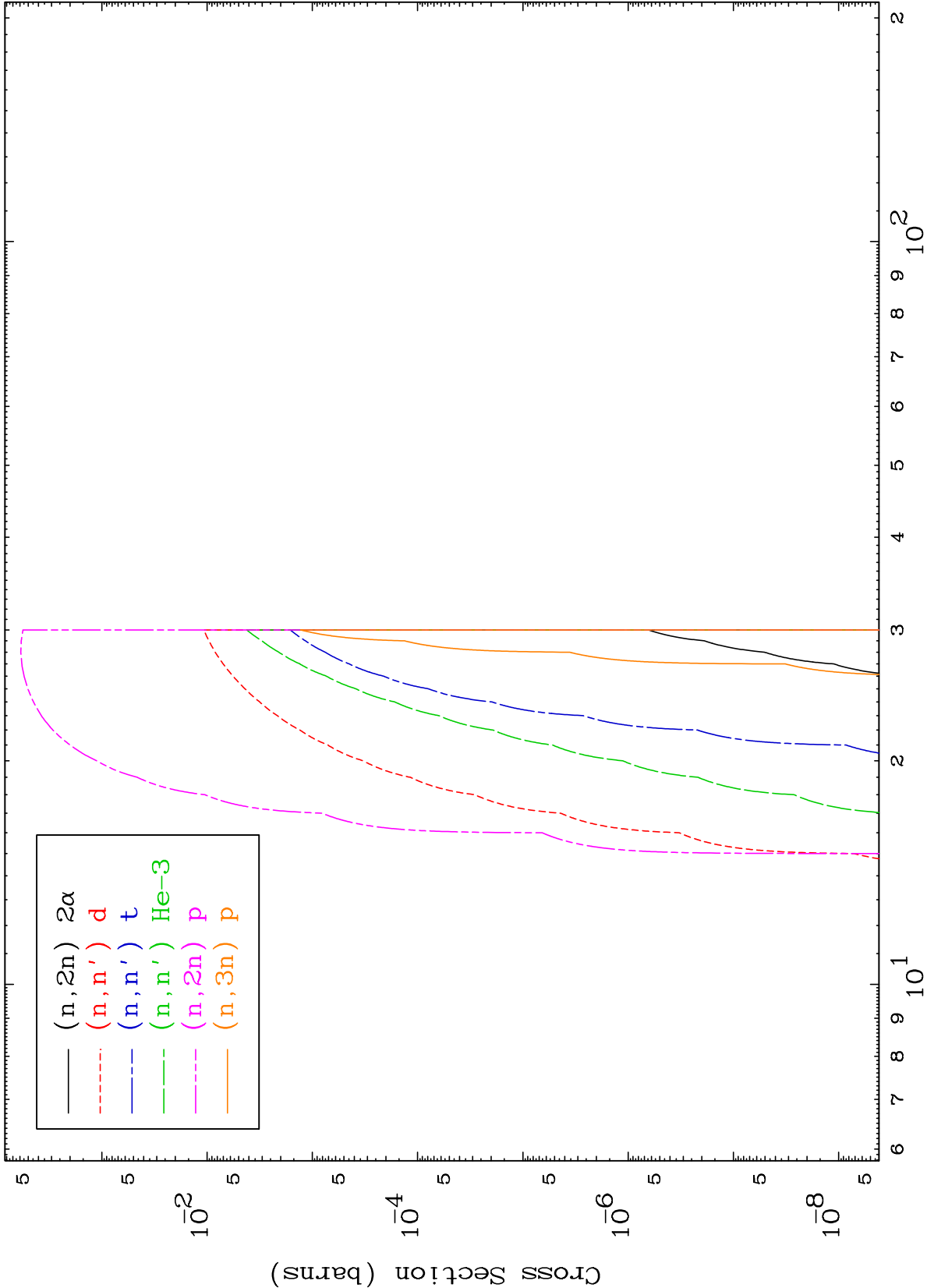
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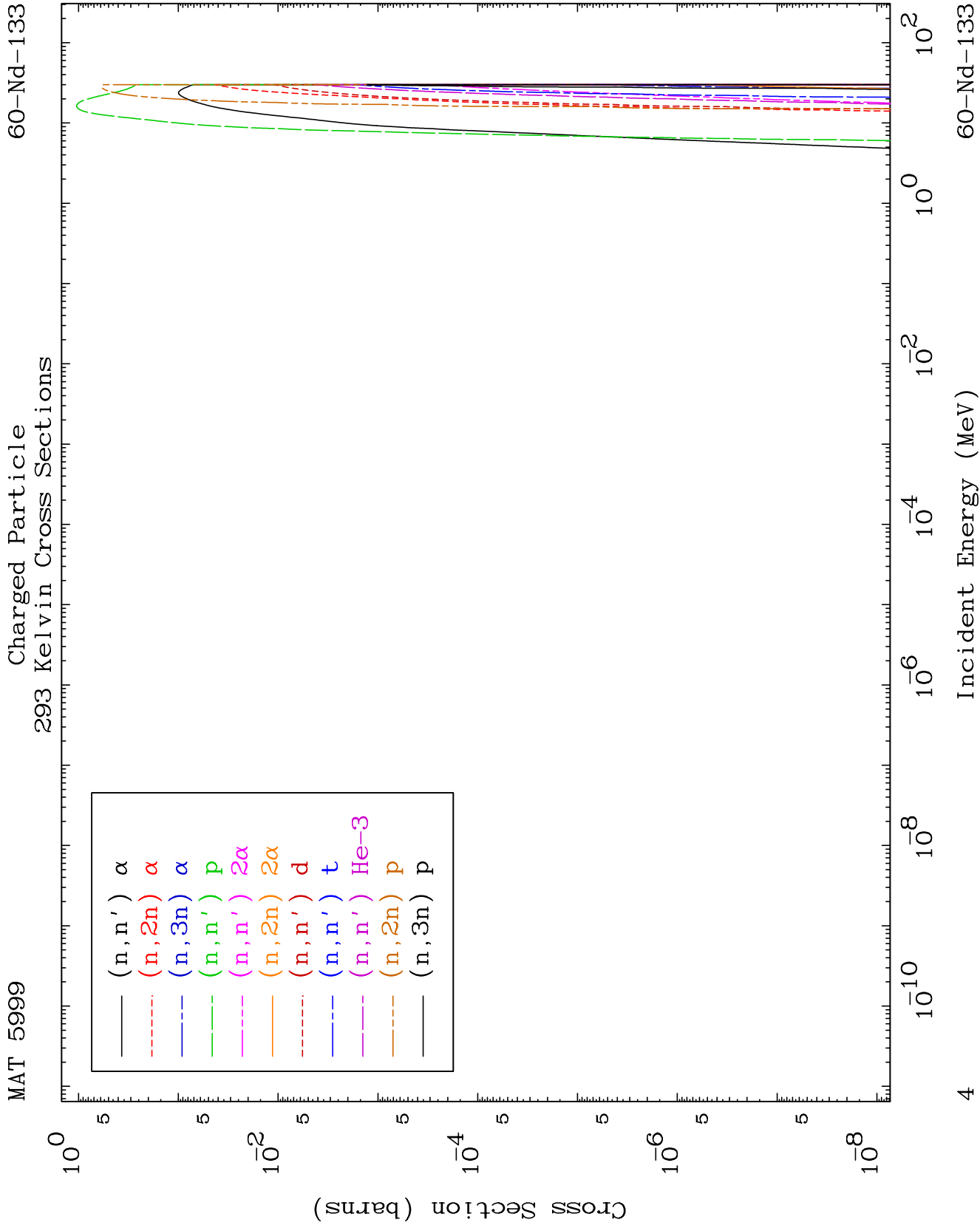
293 Kelvin Cross Sections

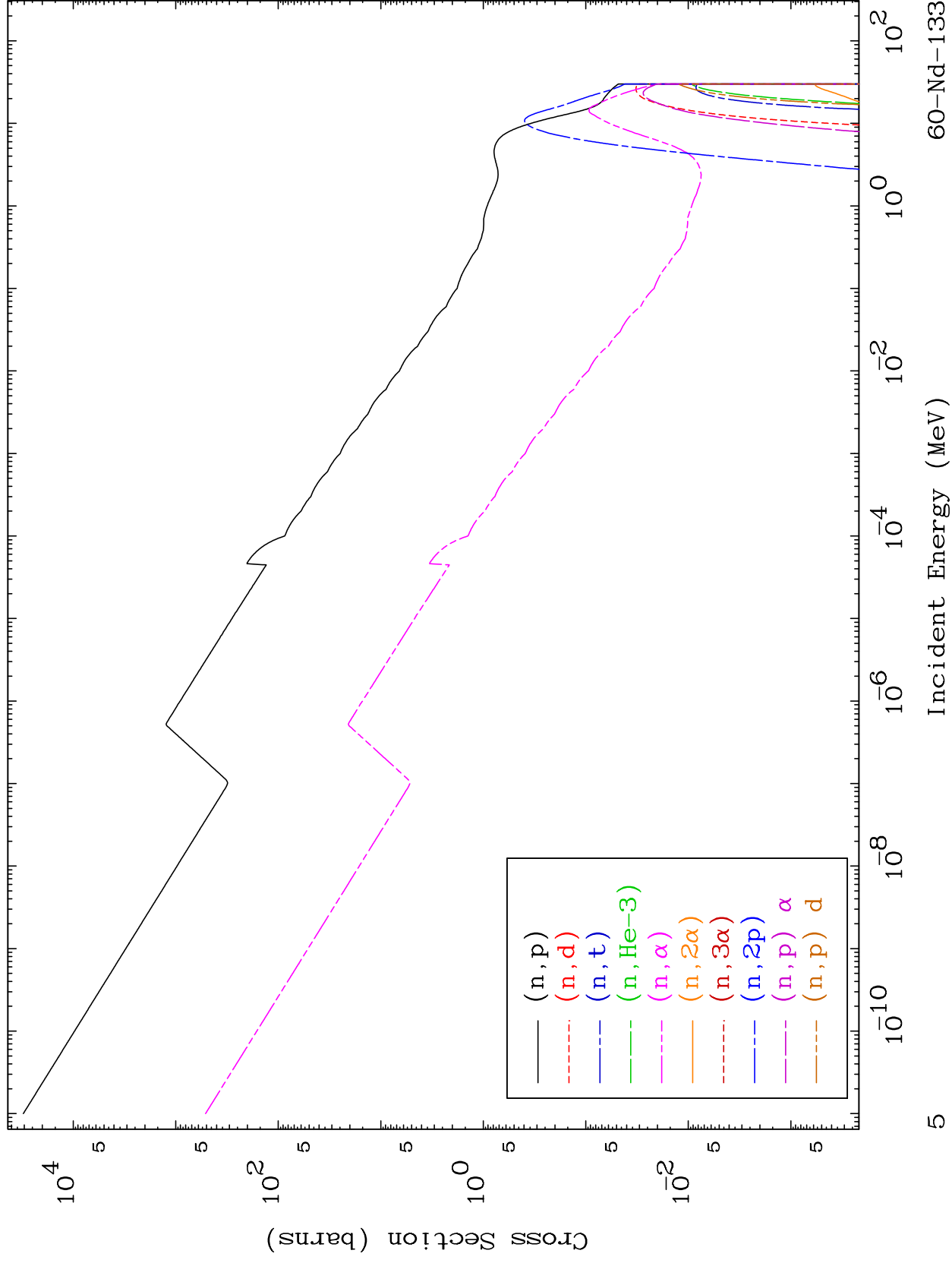
60-Nd-133

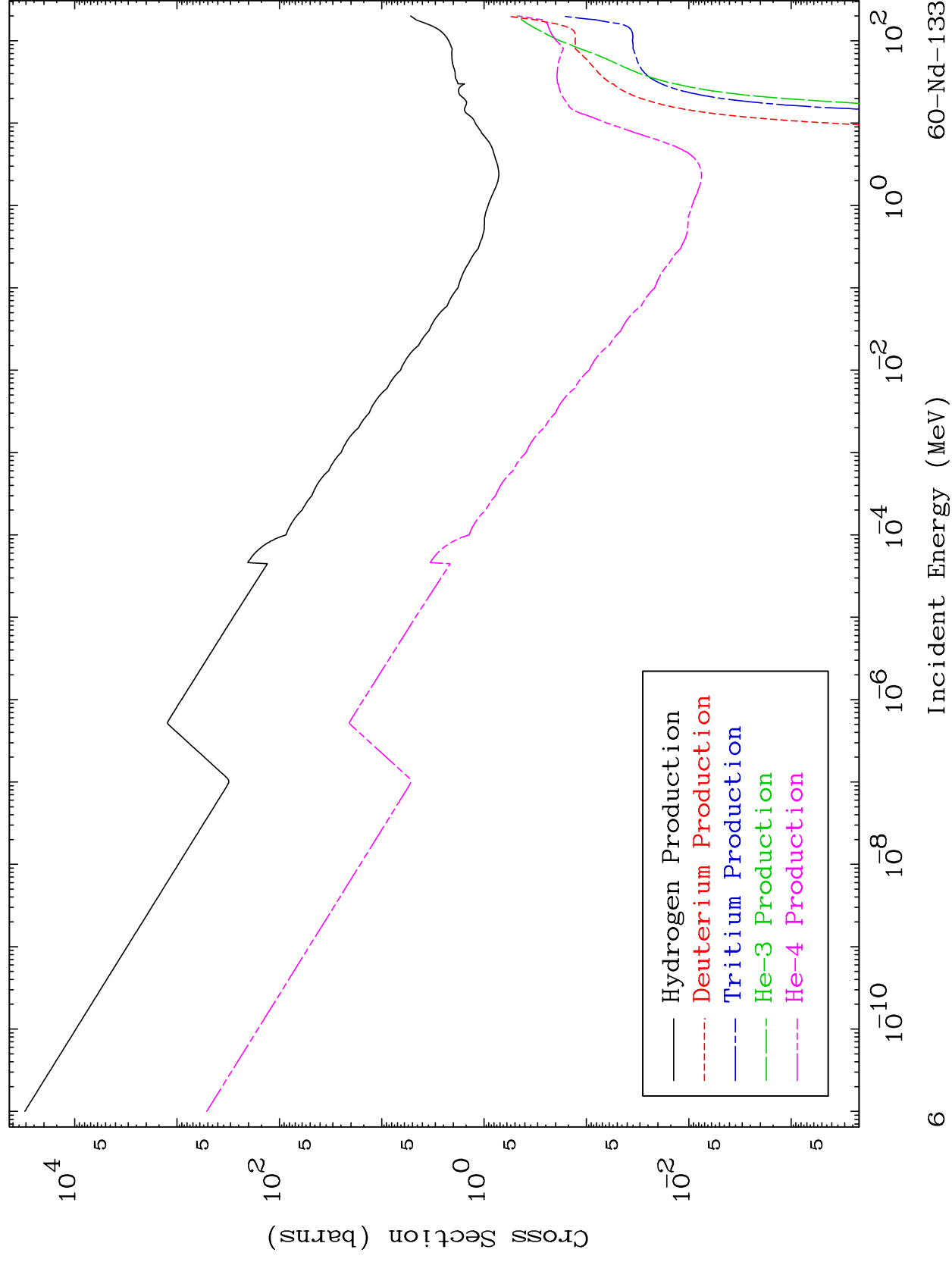


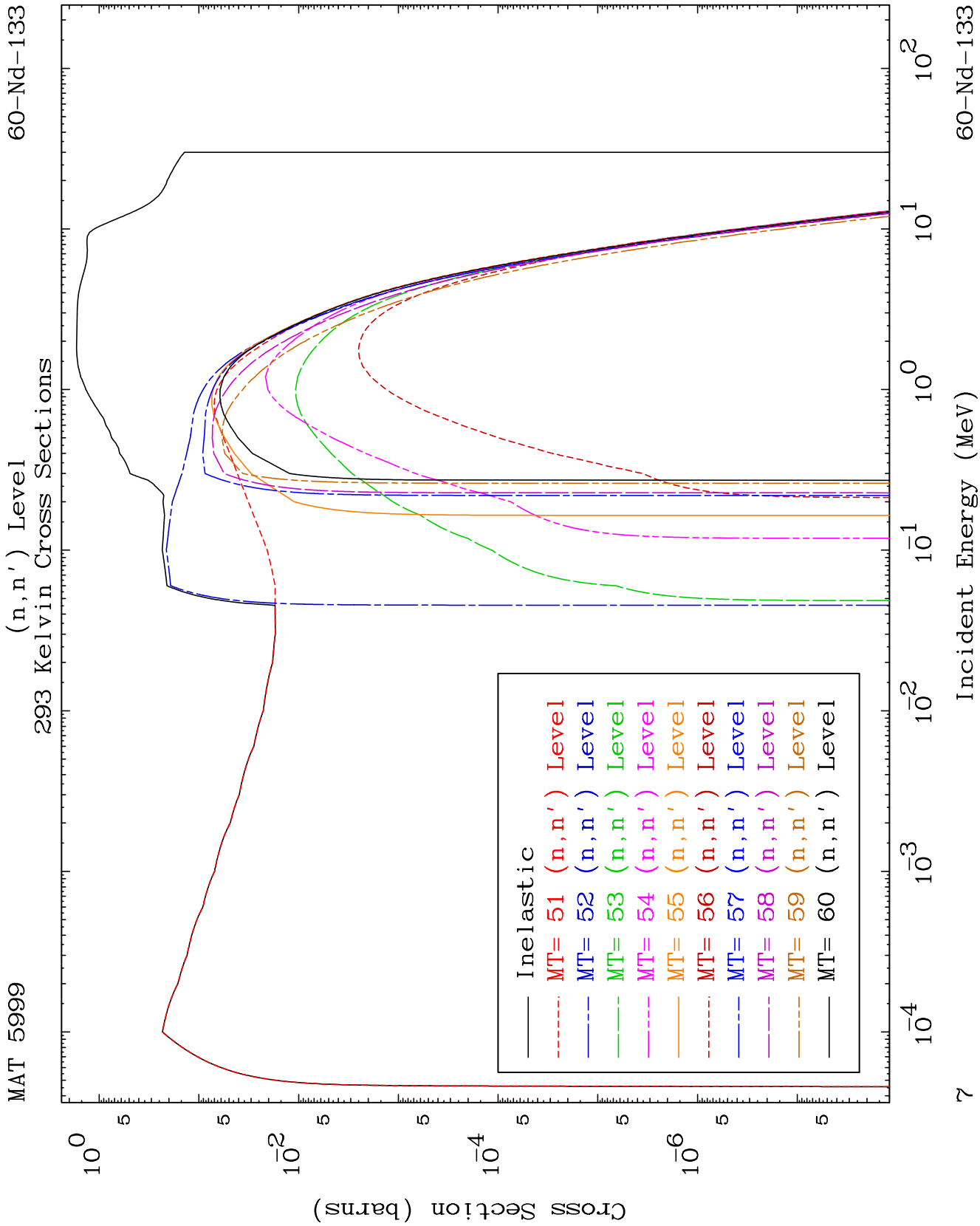


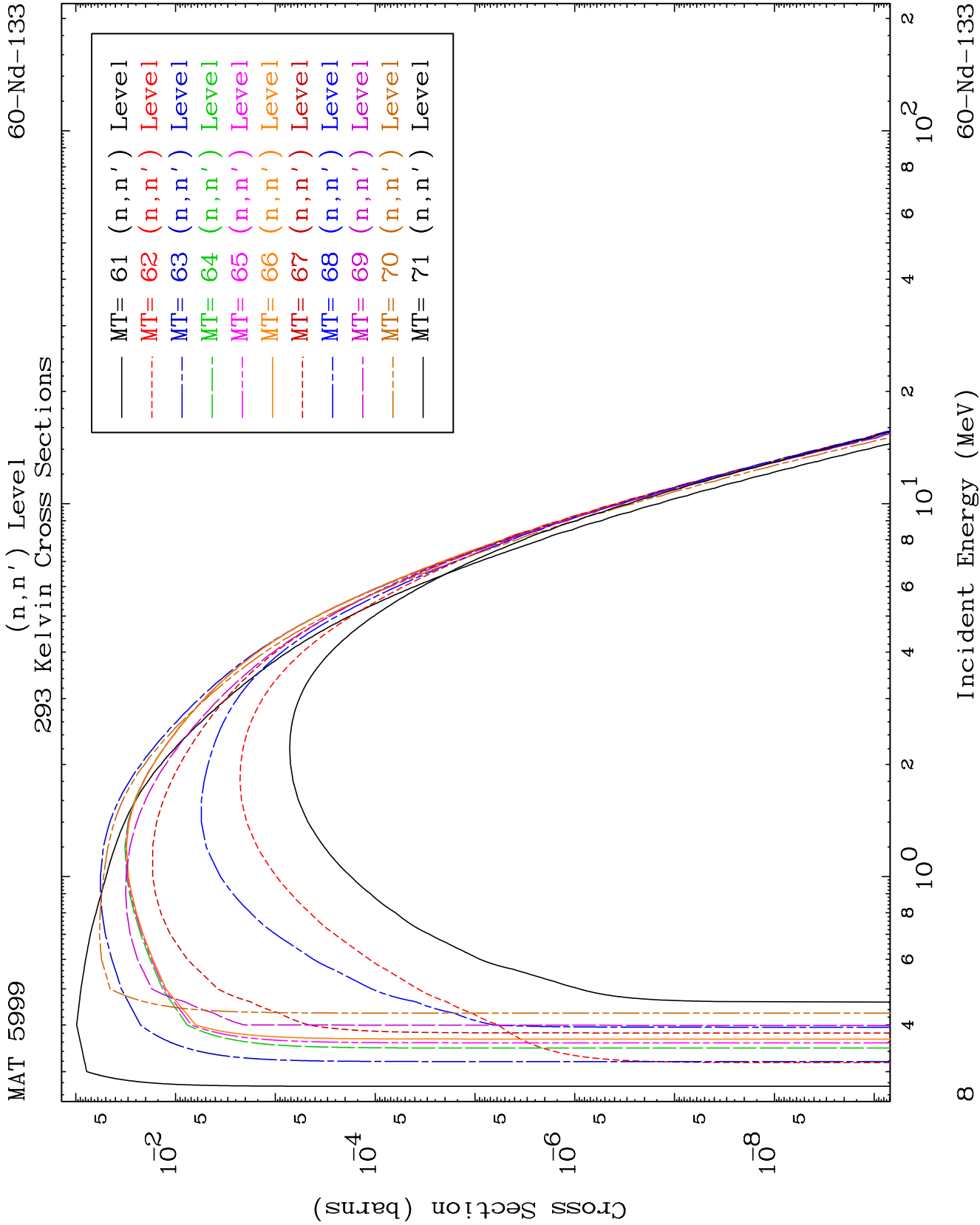


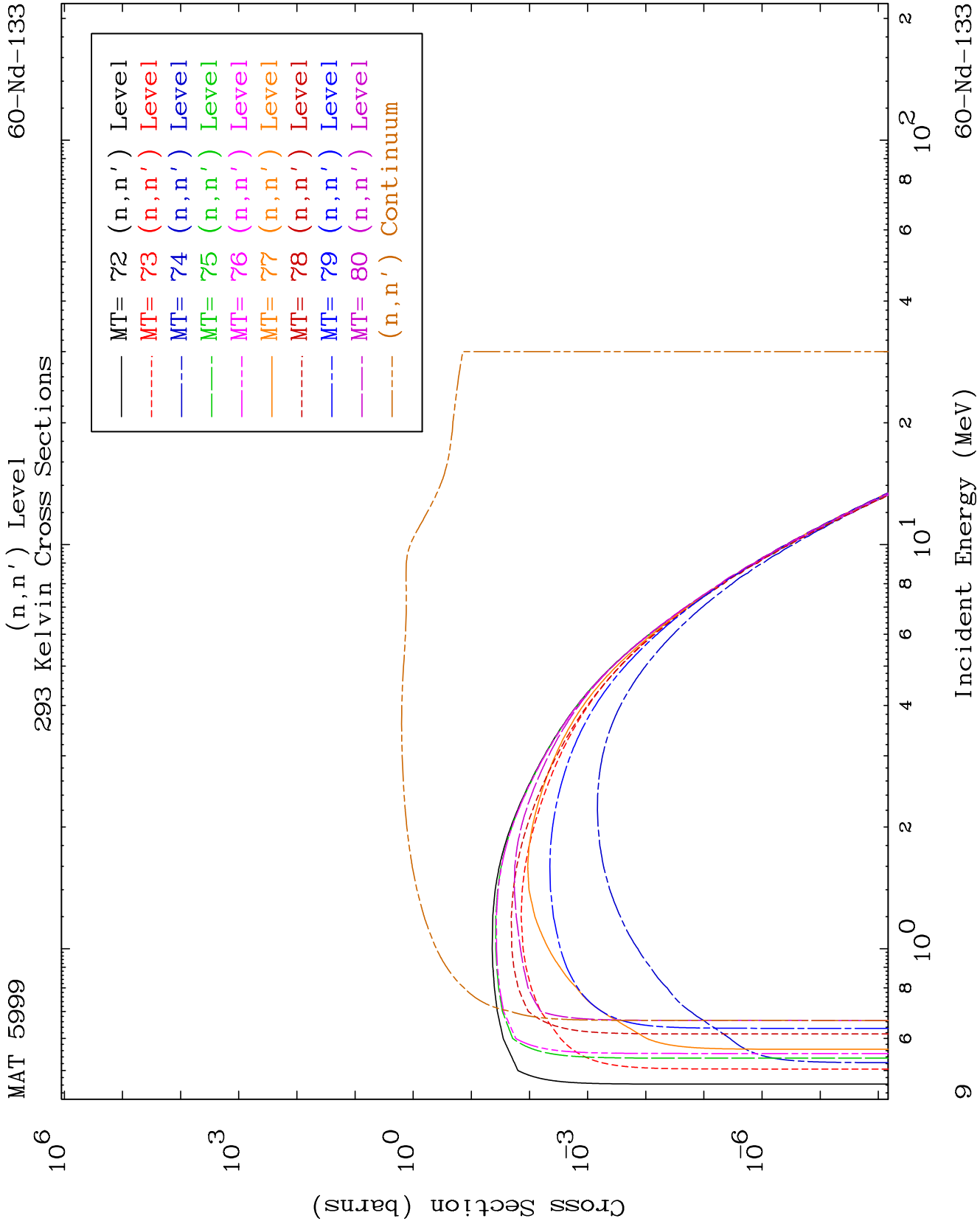


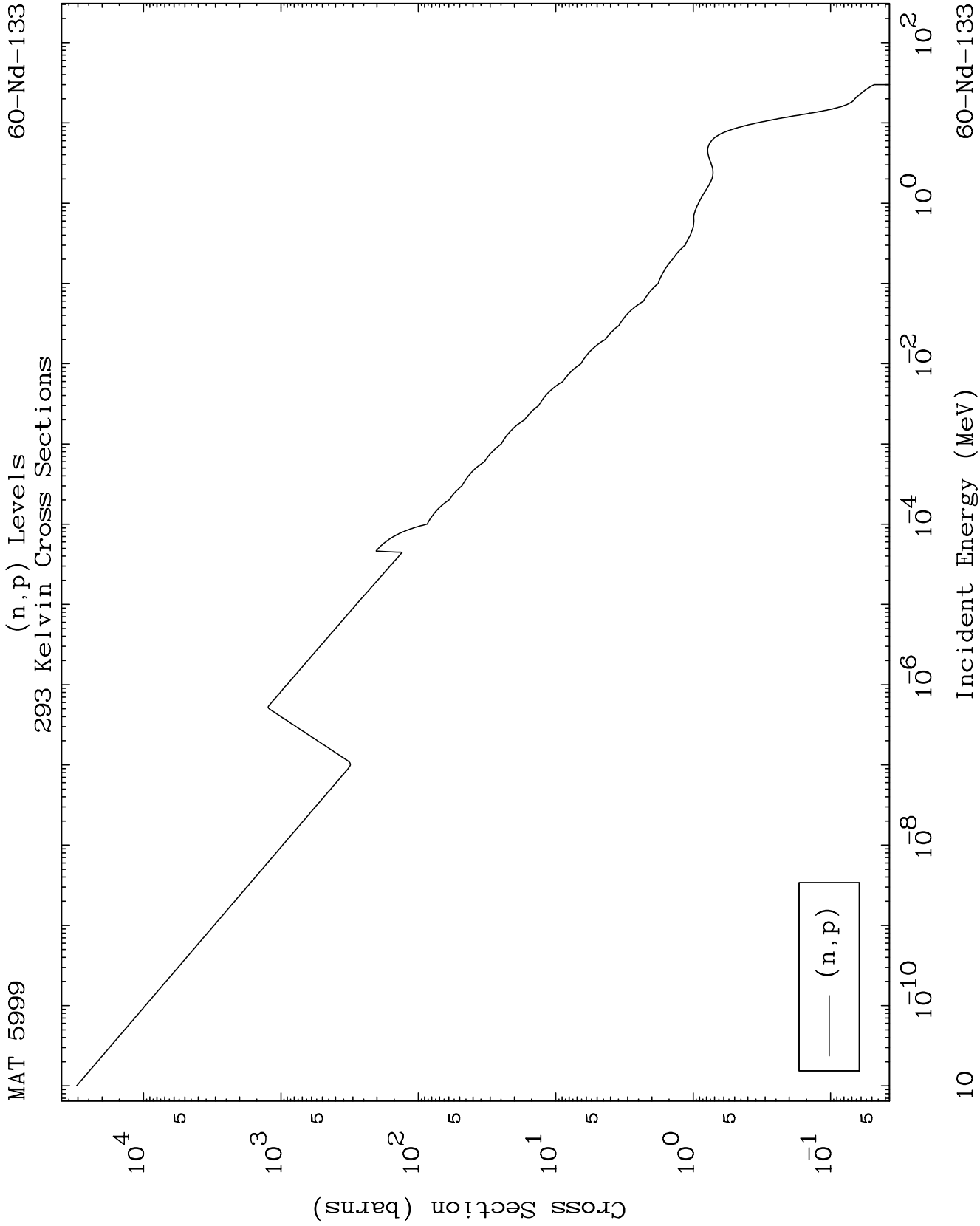








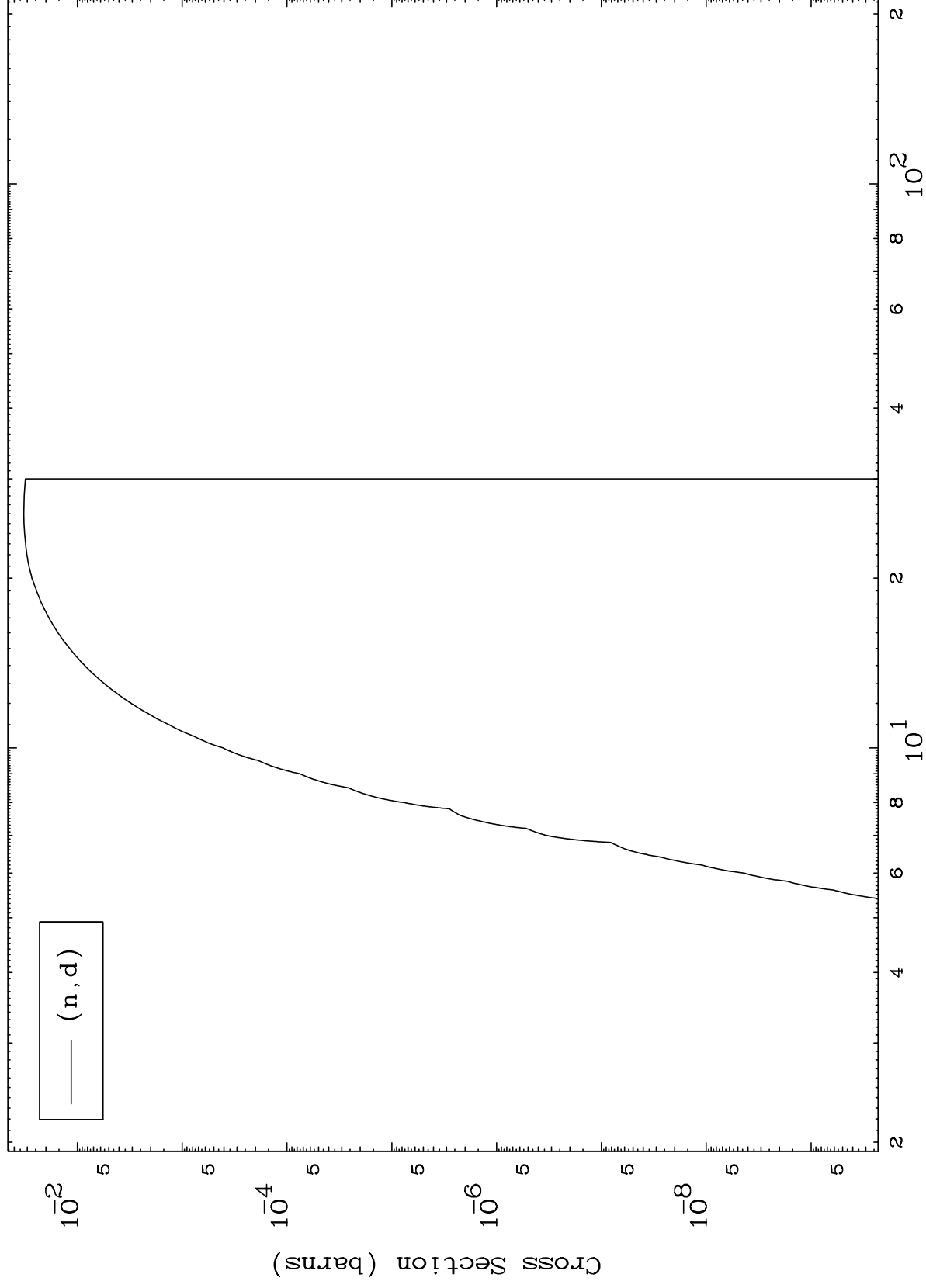




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(n,d) Levels
293 Kelvin Cross Sections

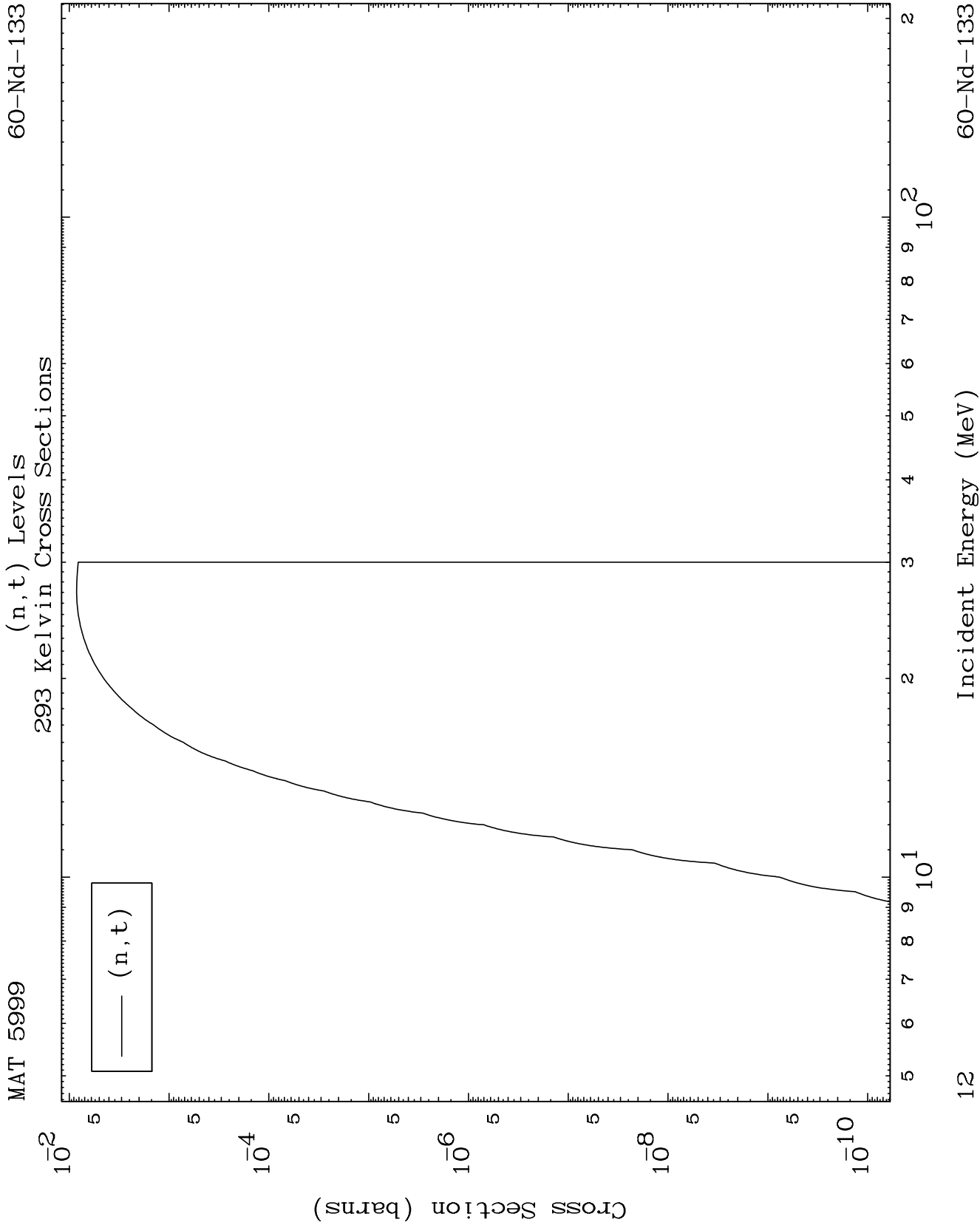
60-Nd-133

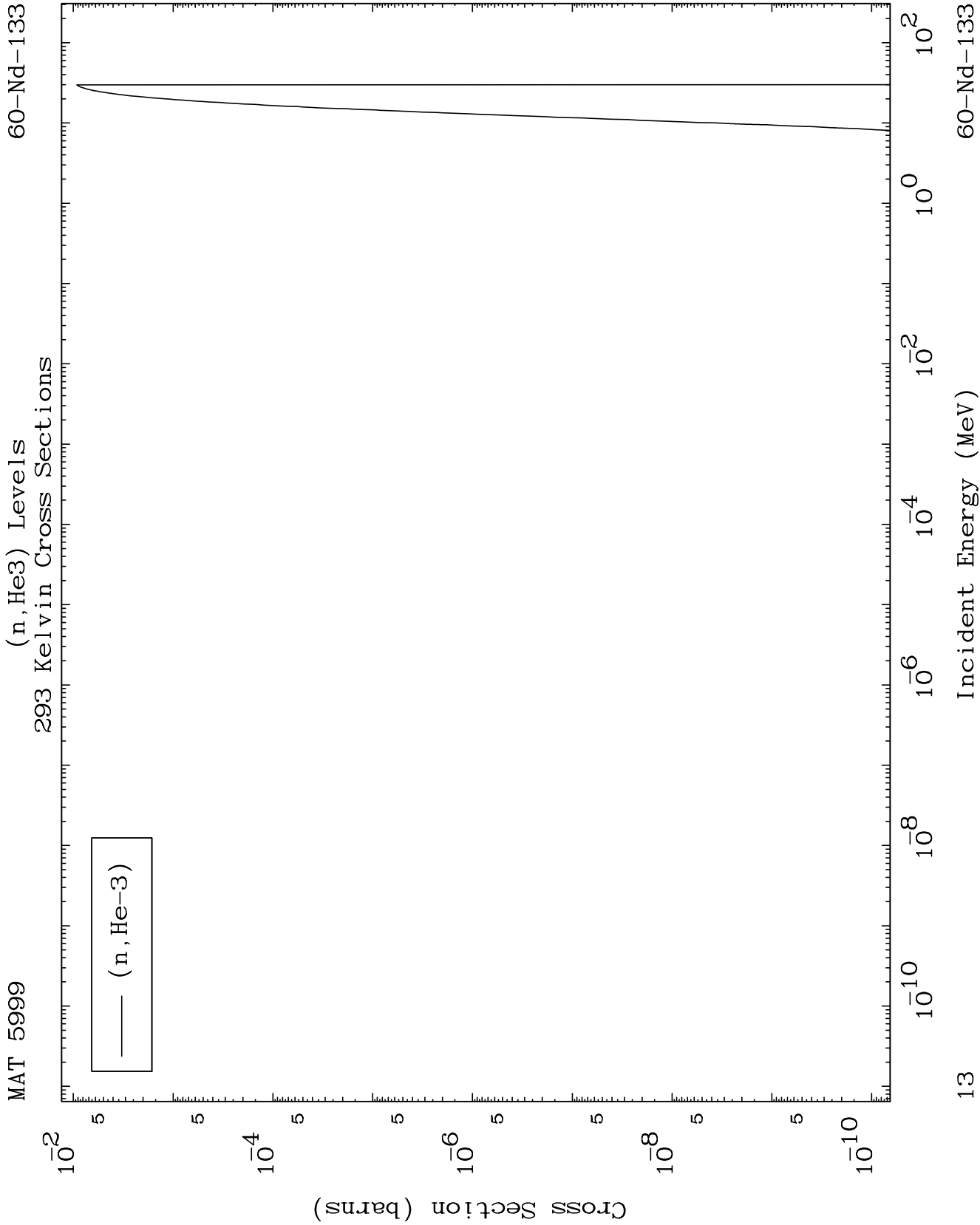


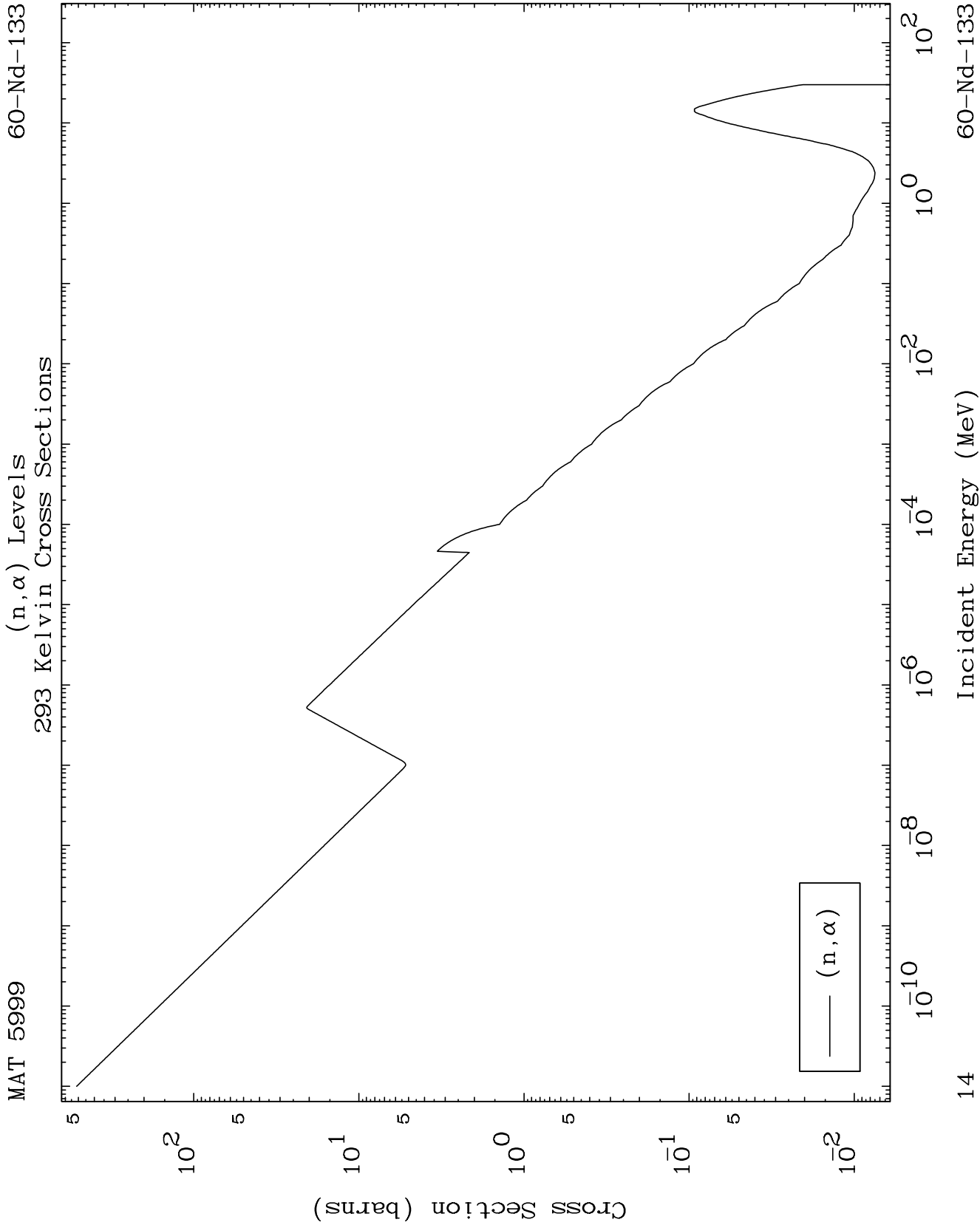
11

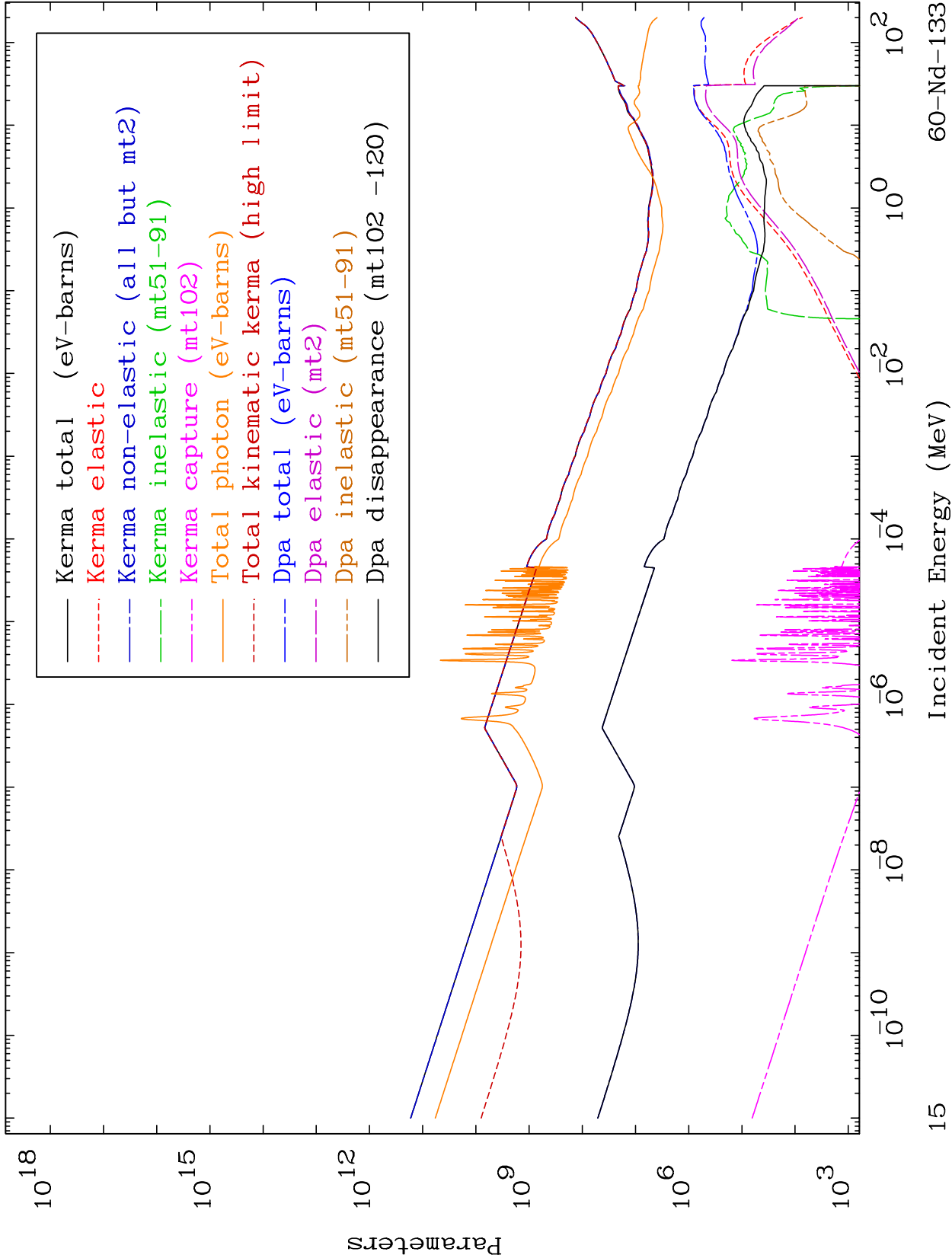
Incident Energy (MeV)

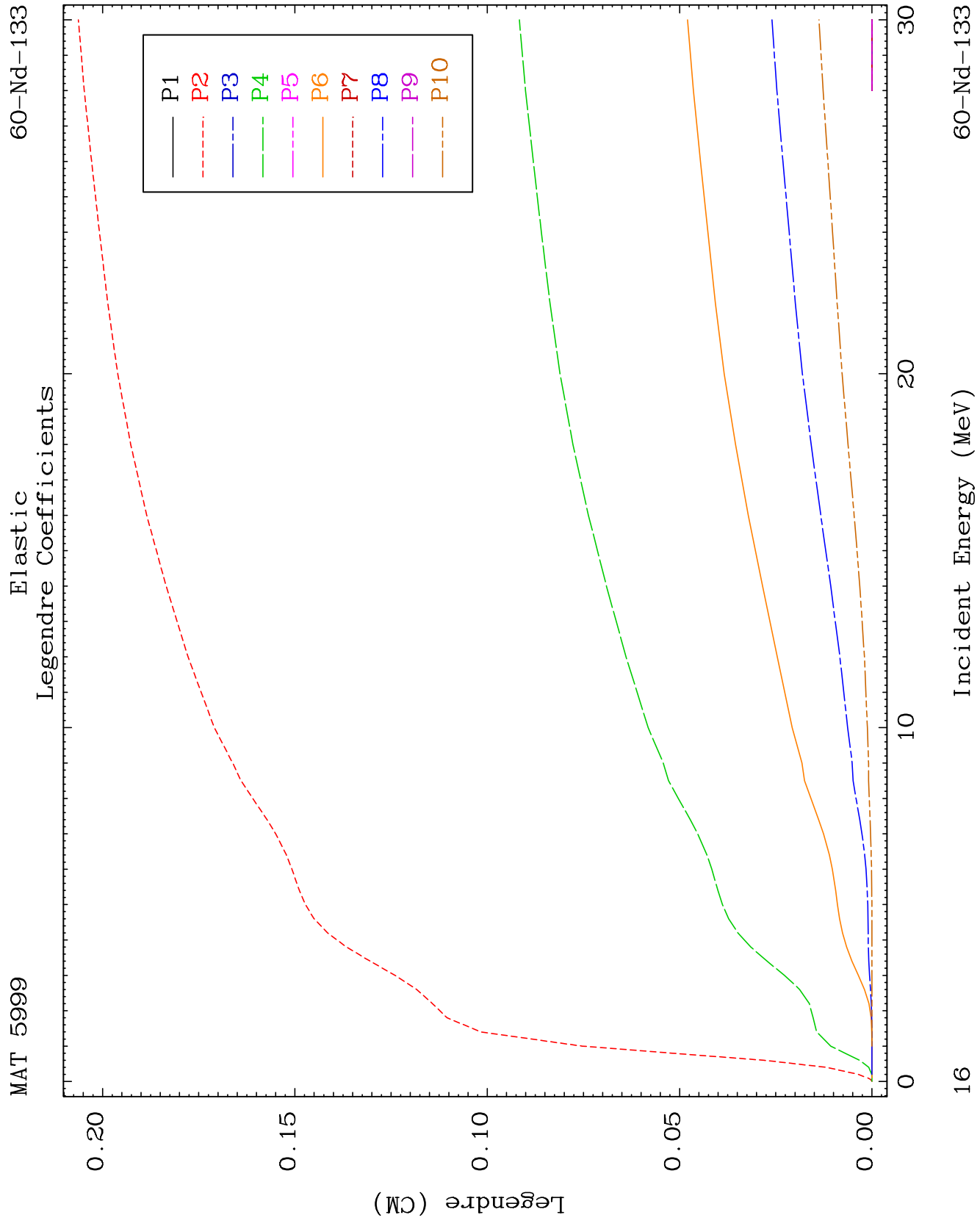
60-Nd-133

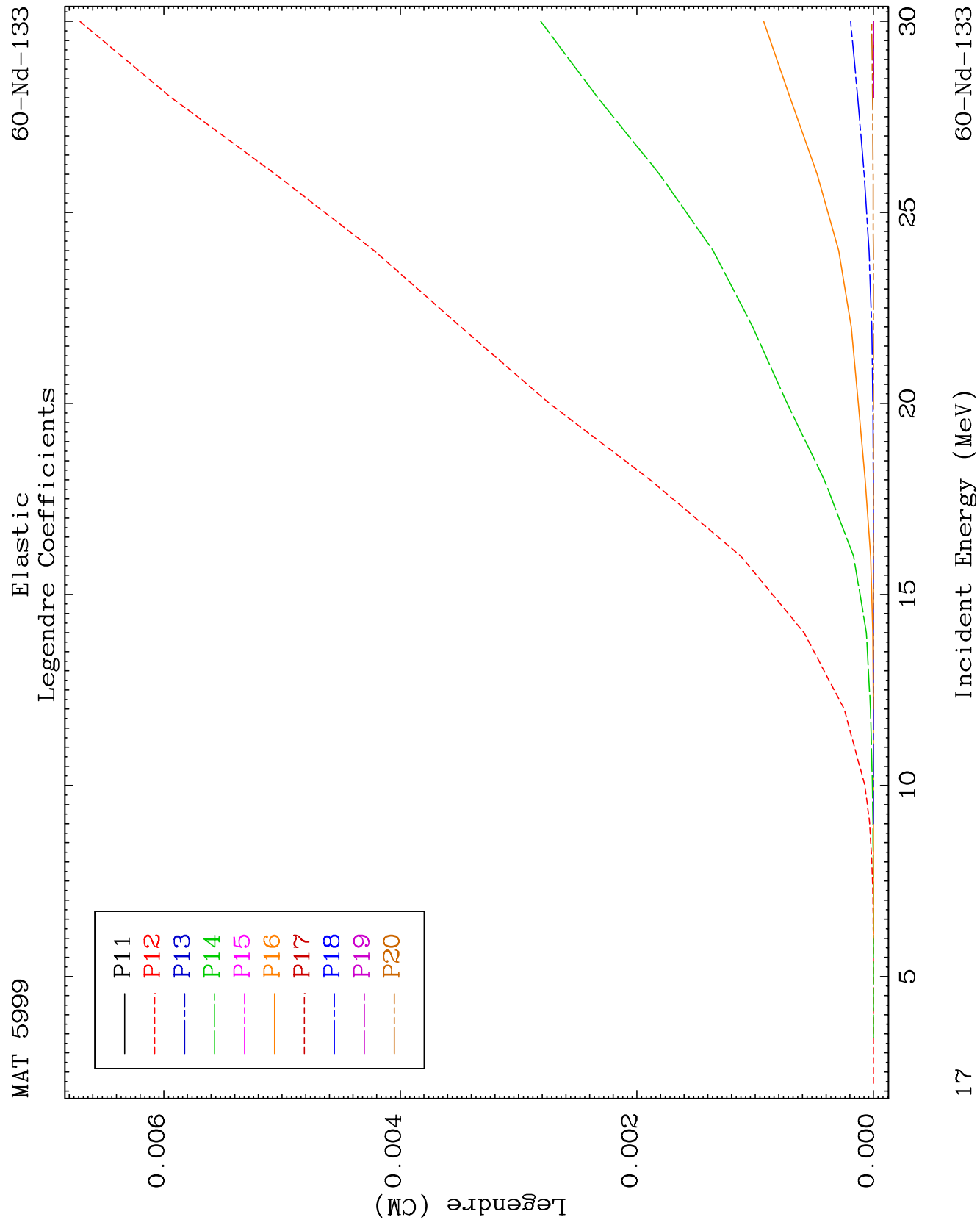


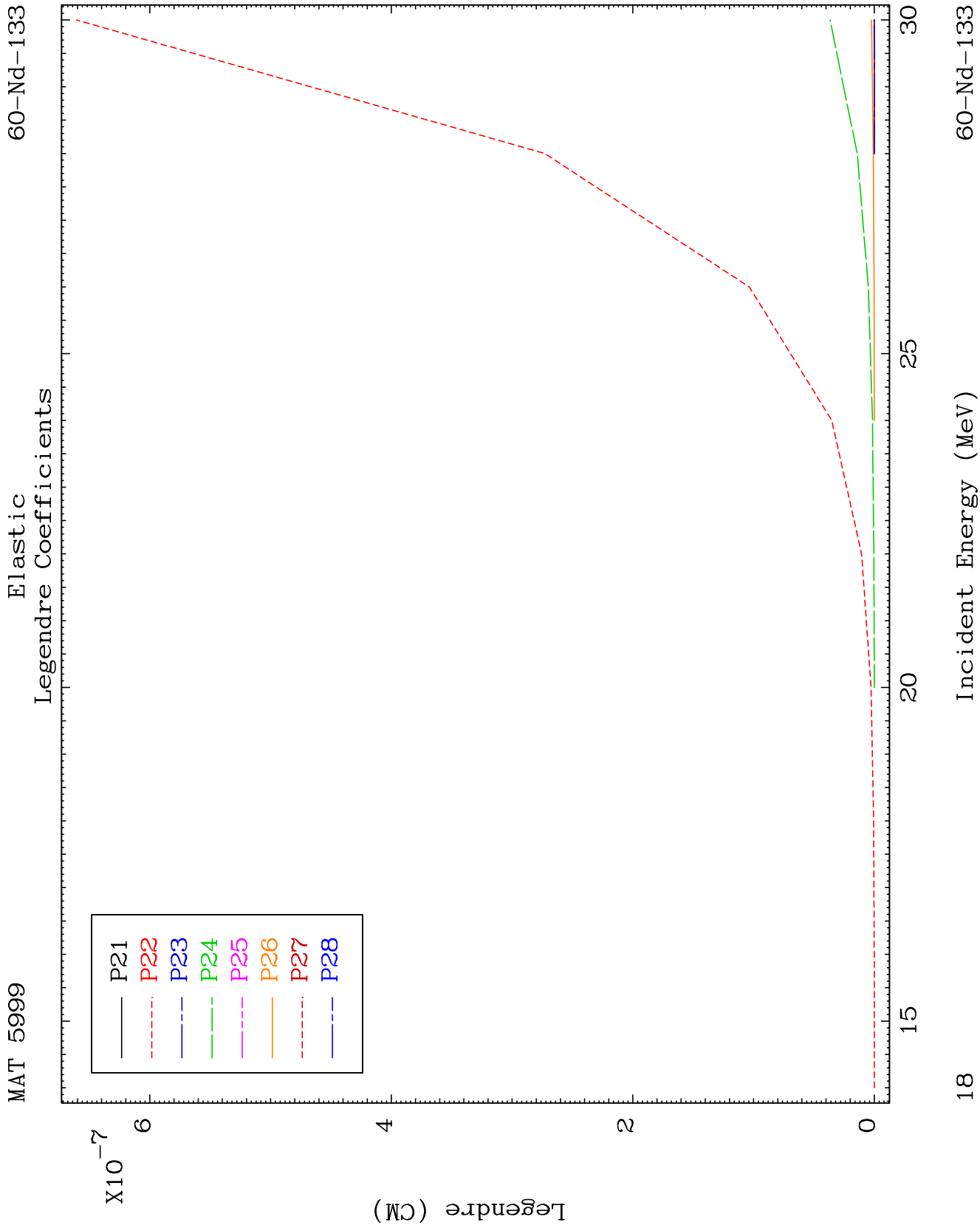


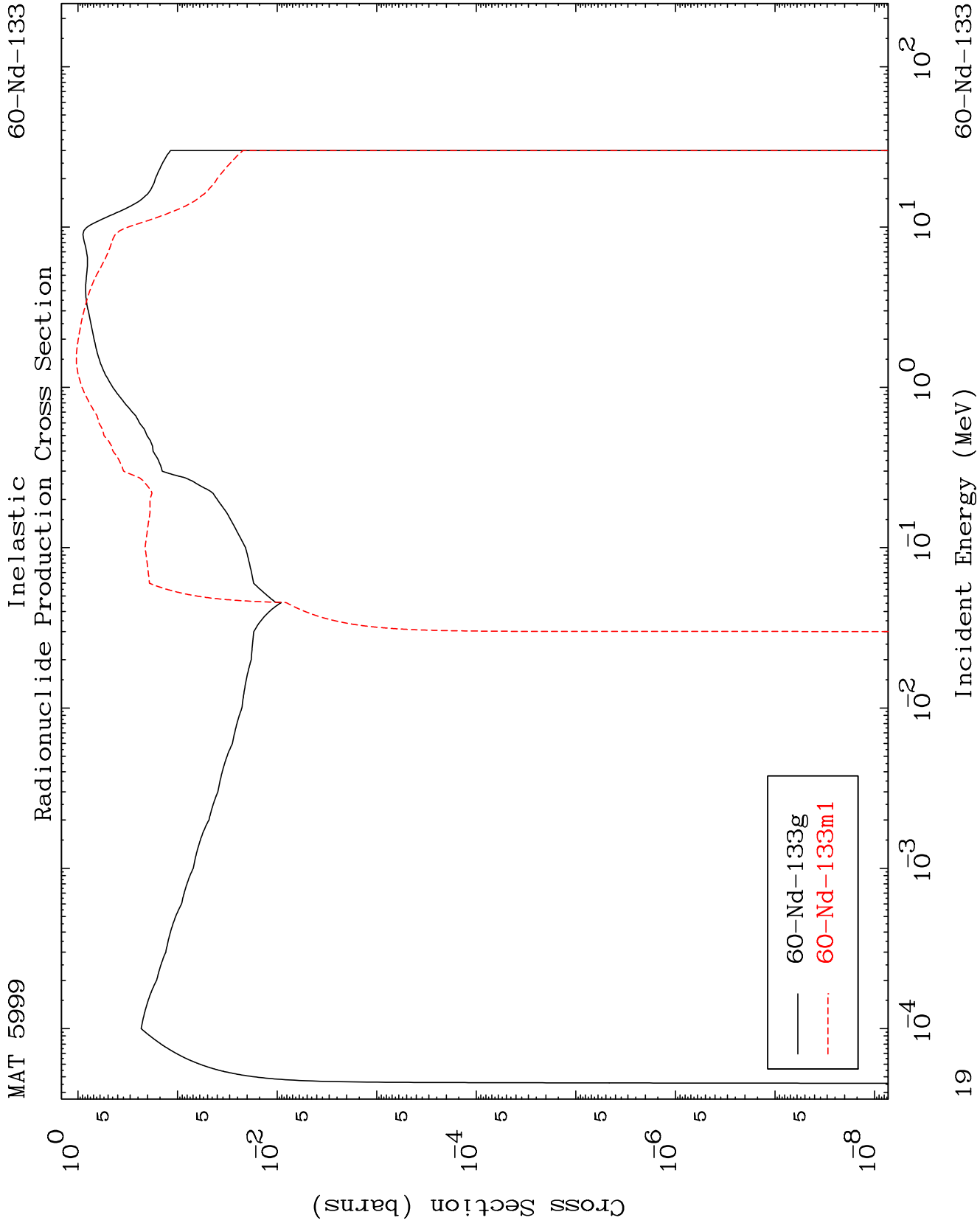


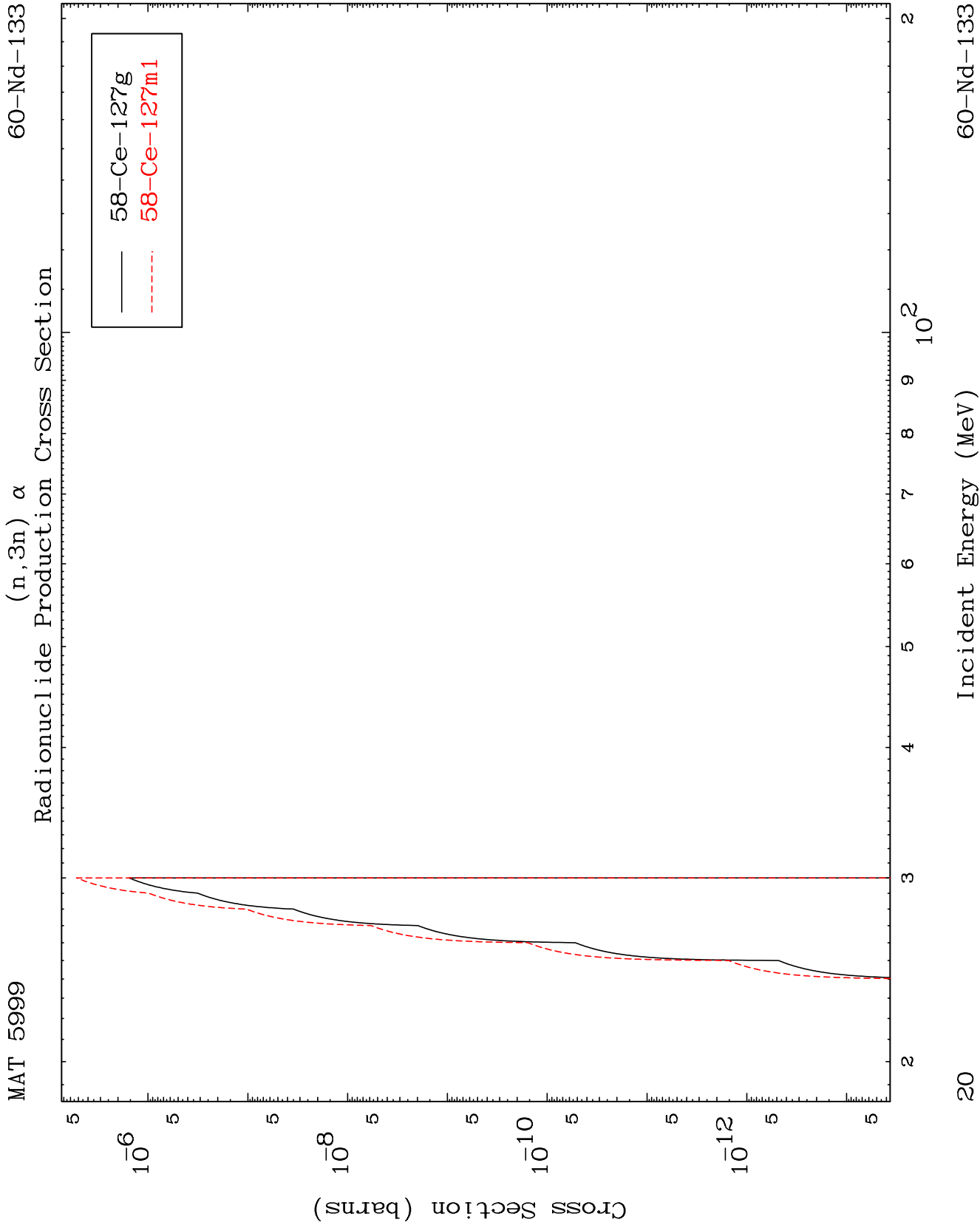


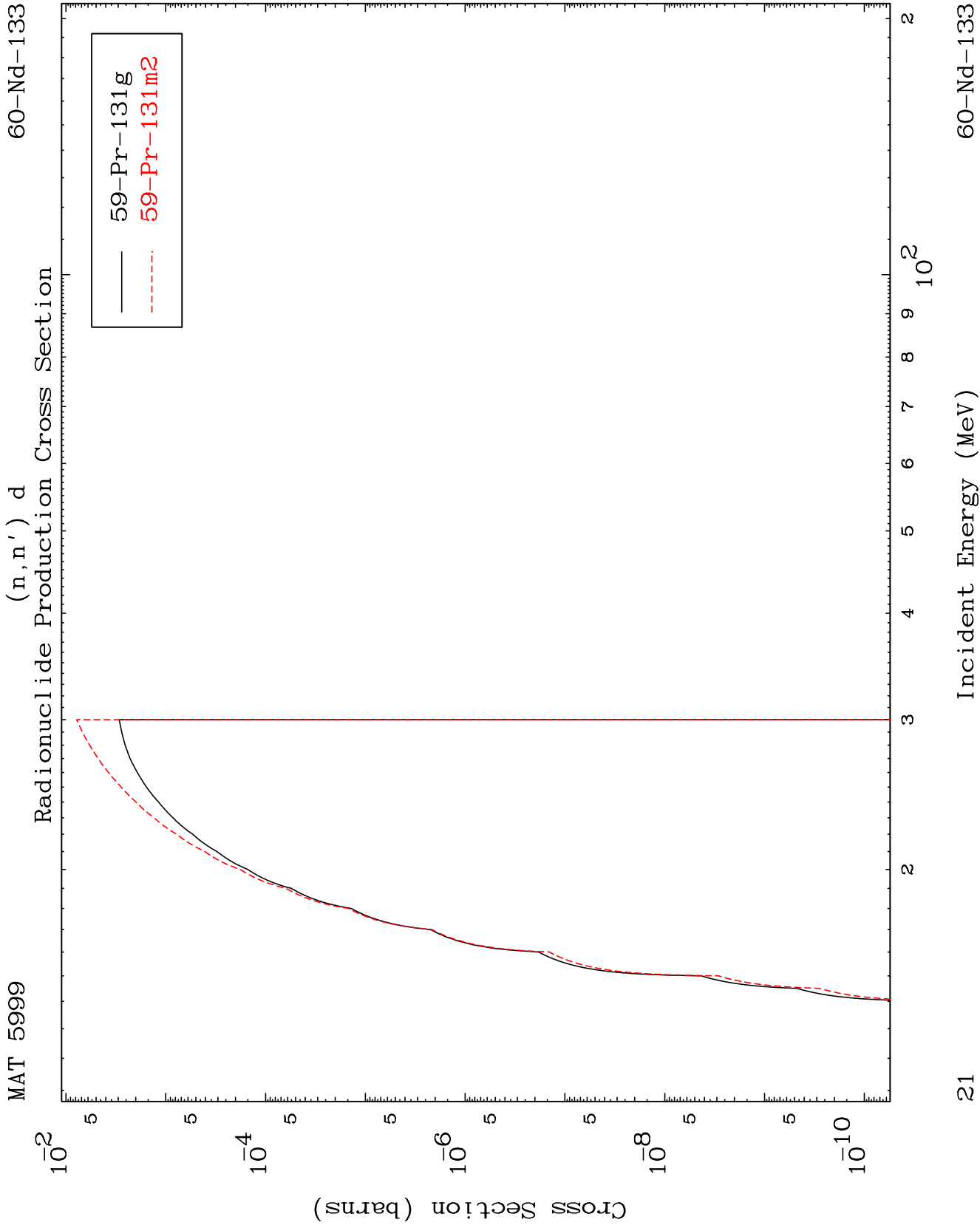








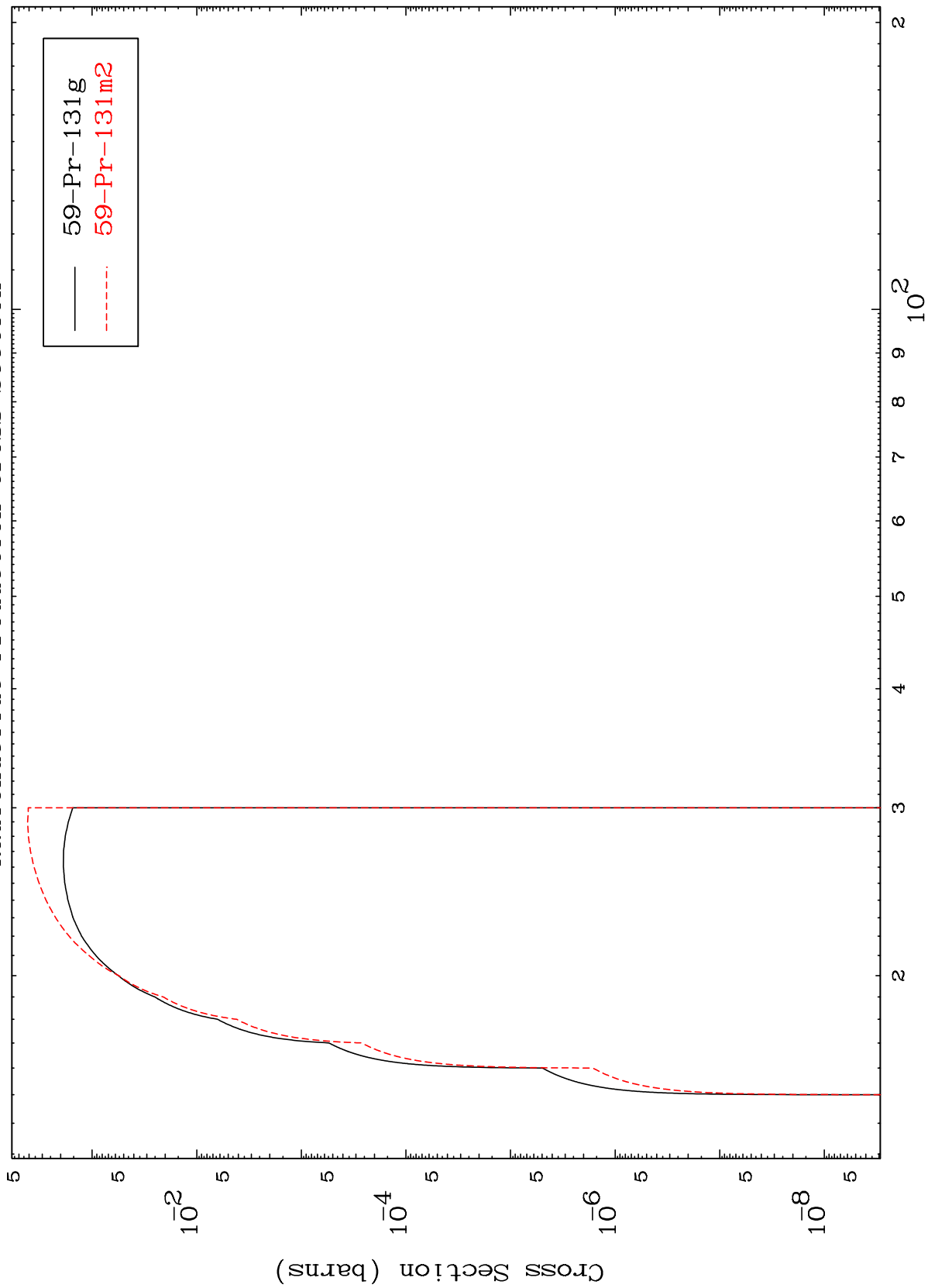




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60-Nd-133

$(n,2n)$ p
Radionuclide Production Cross Section



60-Nd-133

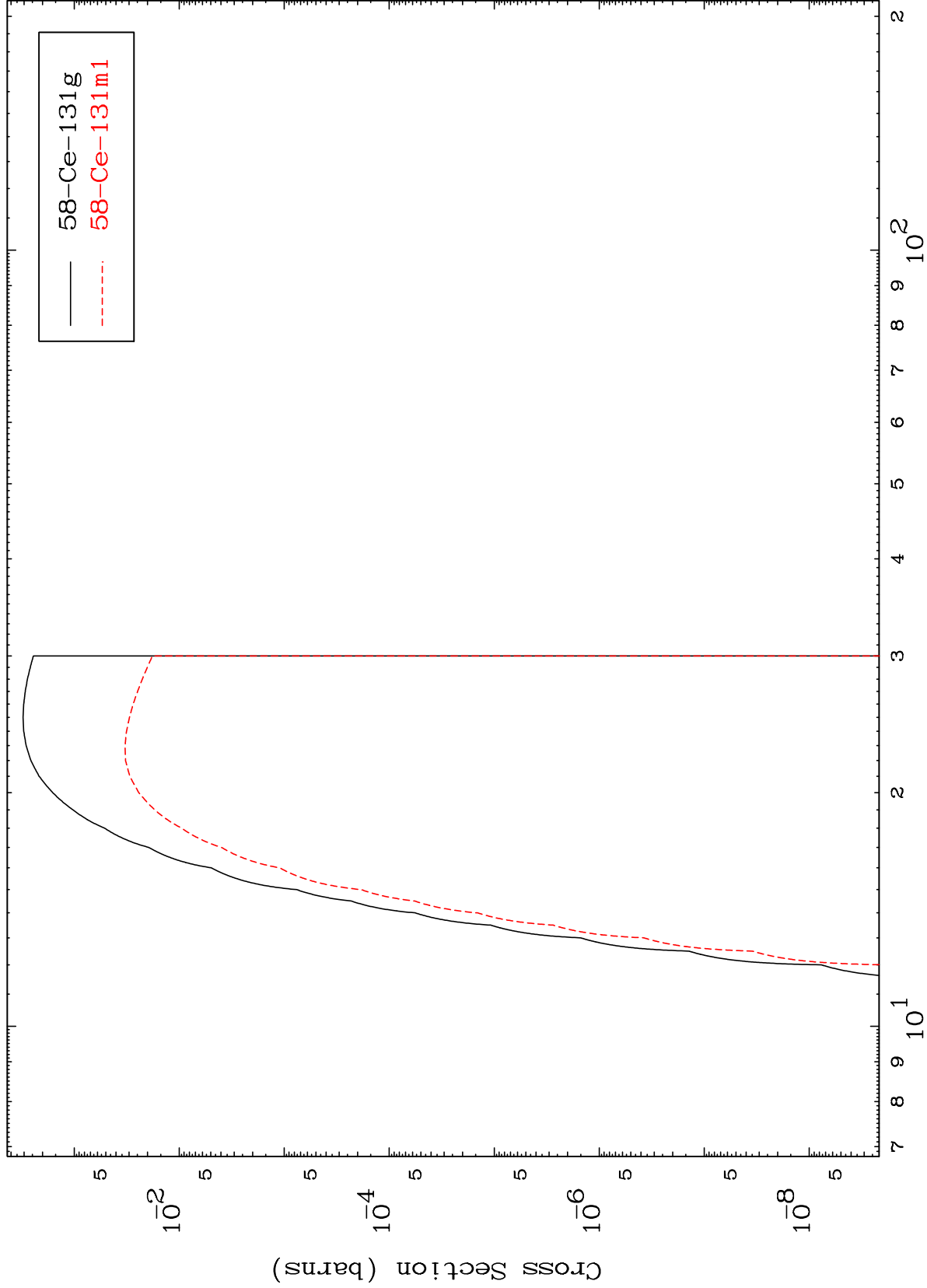
Incident Energy (MeV)

22

MAT 5999

60-Nd-133

(n,2n) p
Radionuclide Production Cross Section



23

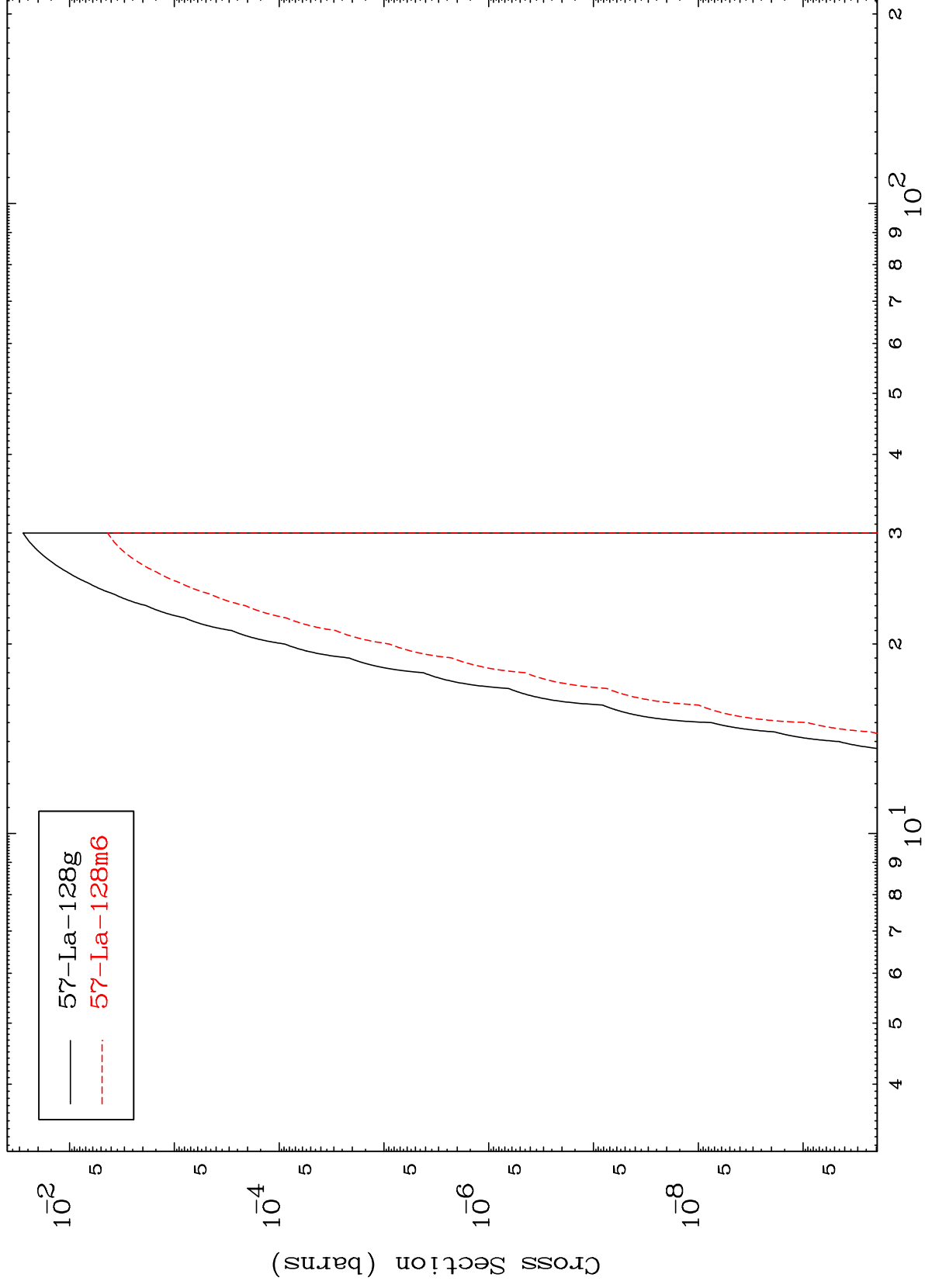
Incident Energy (MeV)

60-Nd-133

MAT 5999

60-Nd-133

(n,n') p α
Radionuclide Production Cross Section



24

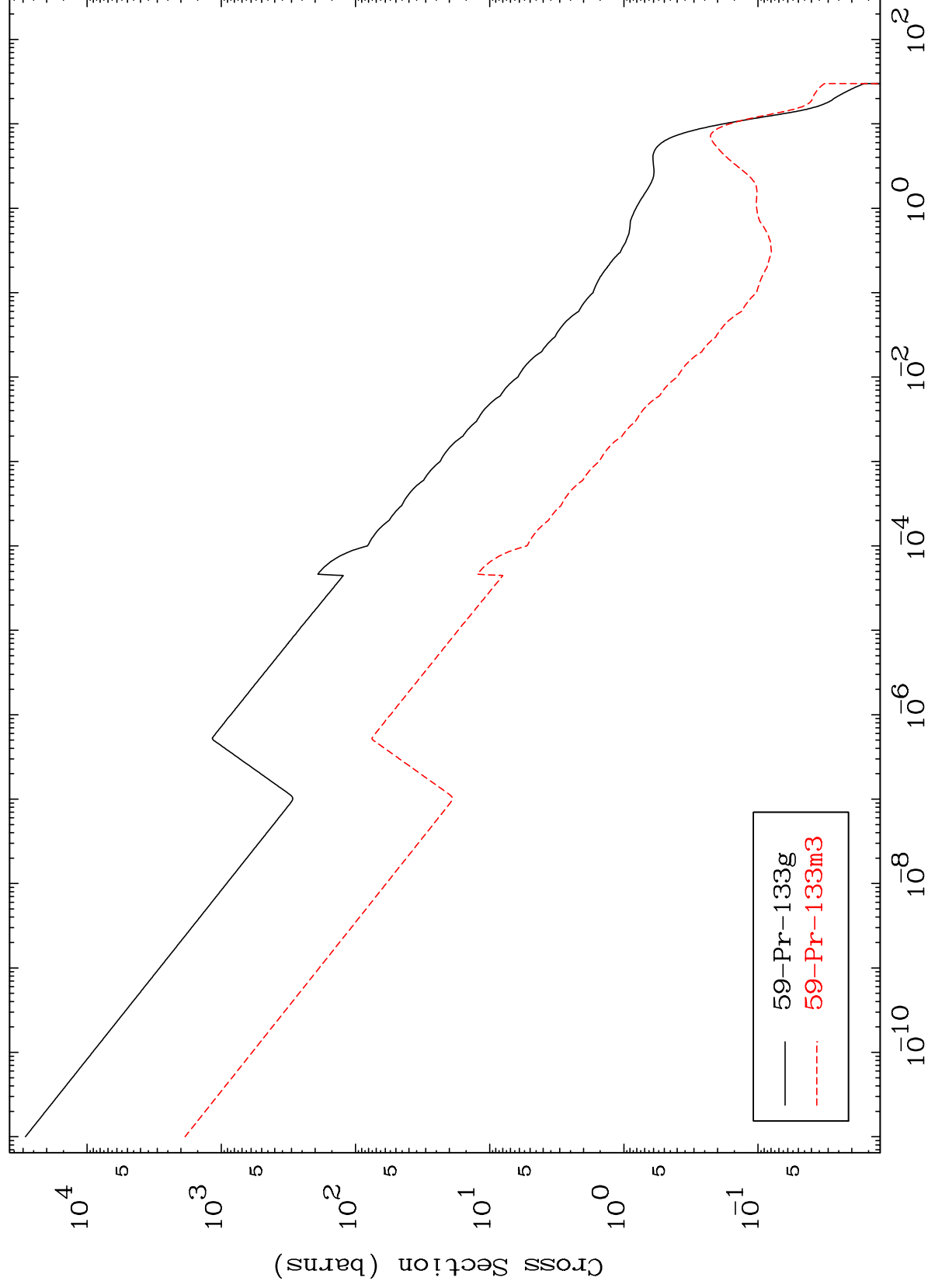
Incident Energy (MeV)

60-Nd-133

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60-Nd-133

(n,p)
Radionuclide Production Cross Section



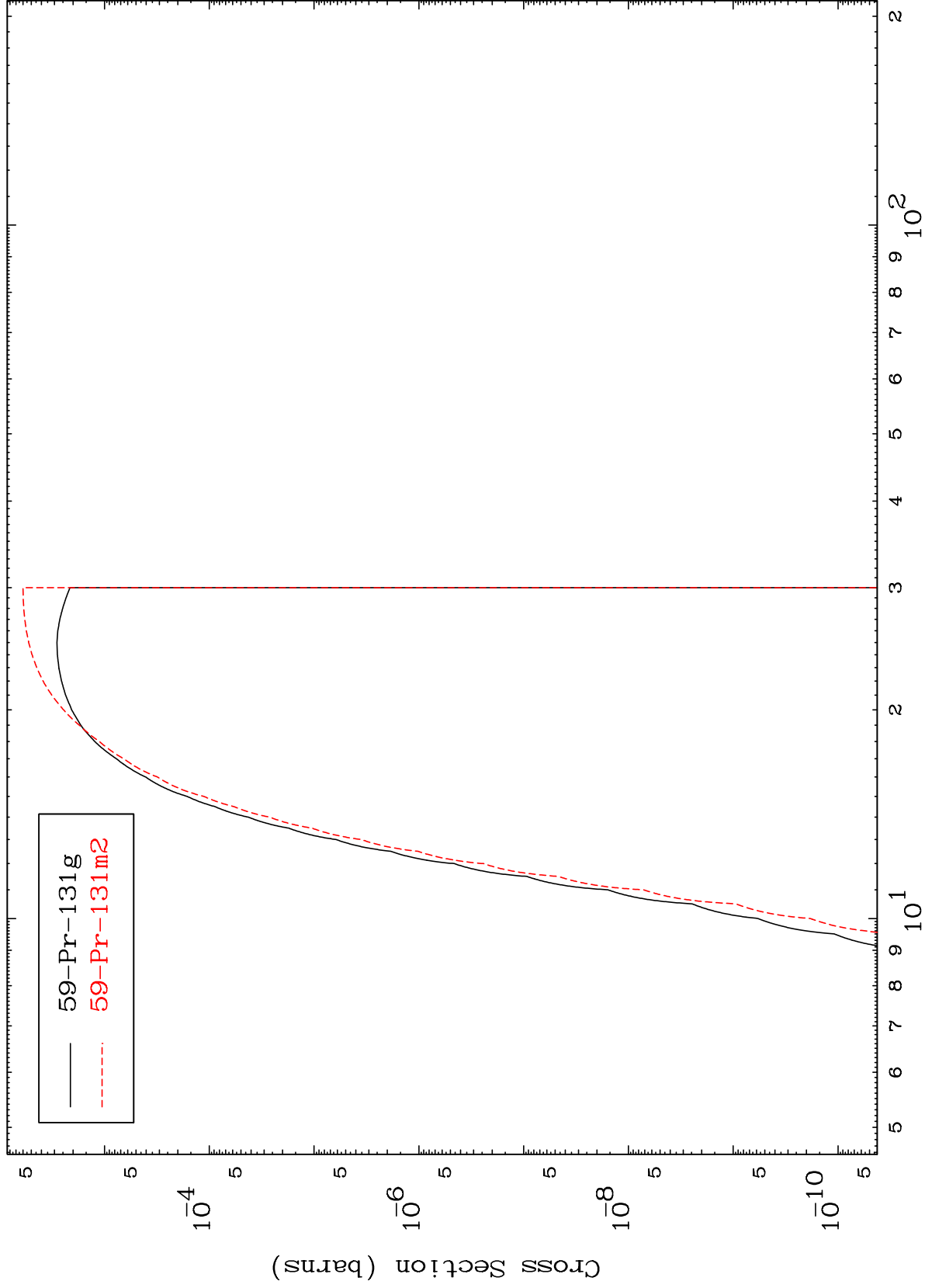
25

60-Nd-133

MAT 5999

60-Nd-133

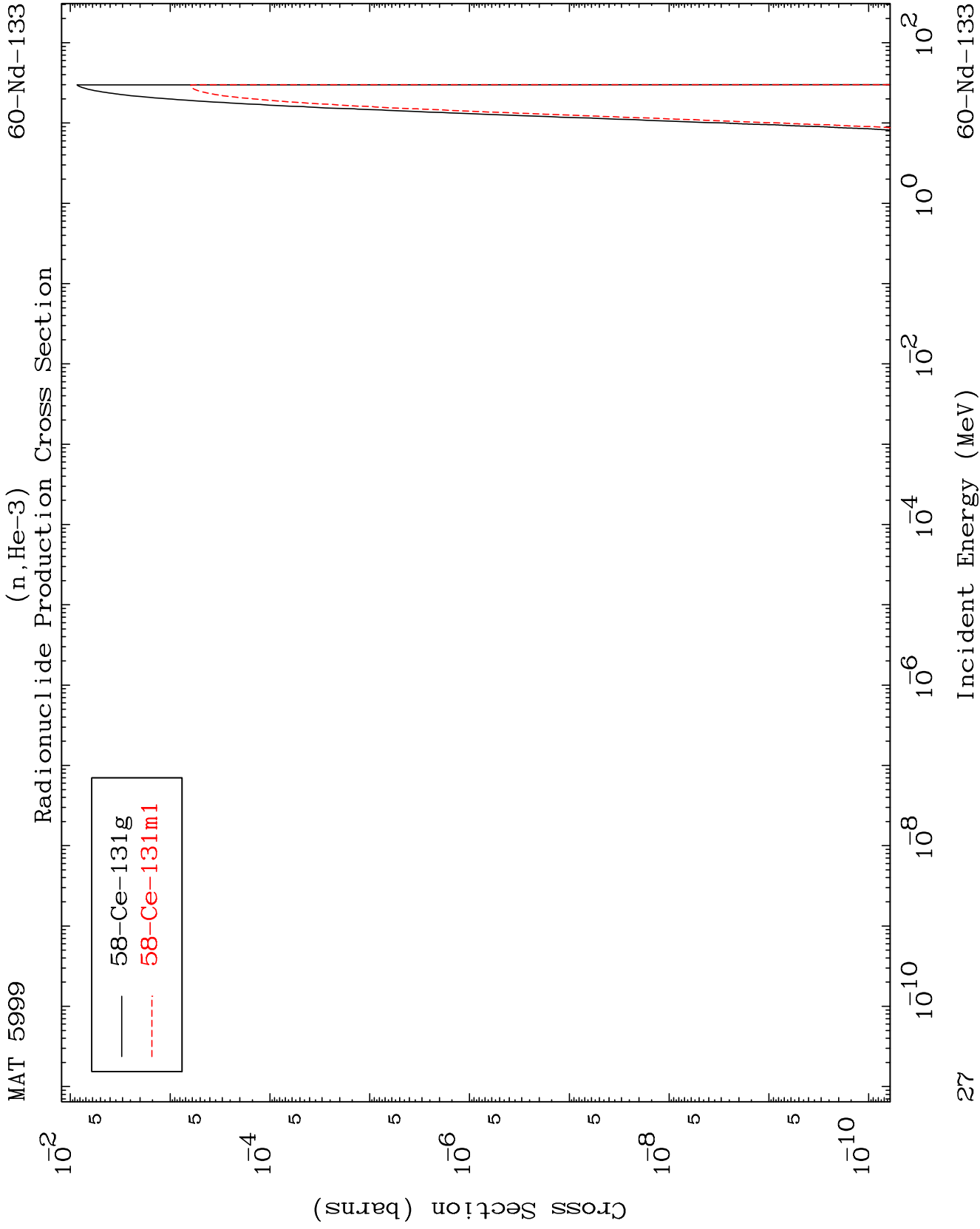
(n,t)
Radionuclide Production Cross Section



26

Incident Energy (MeV)

60-Nd-133

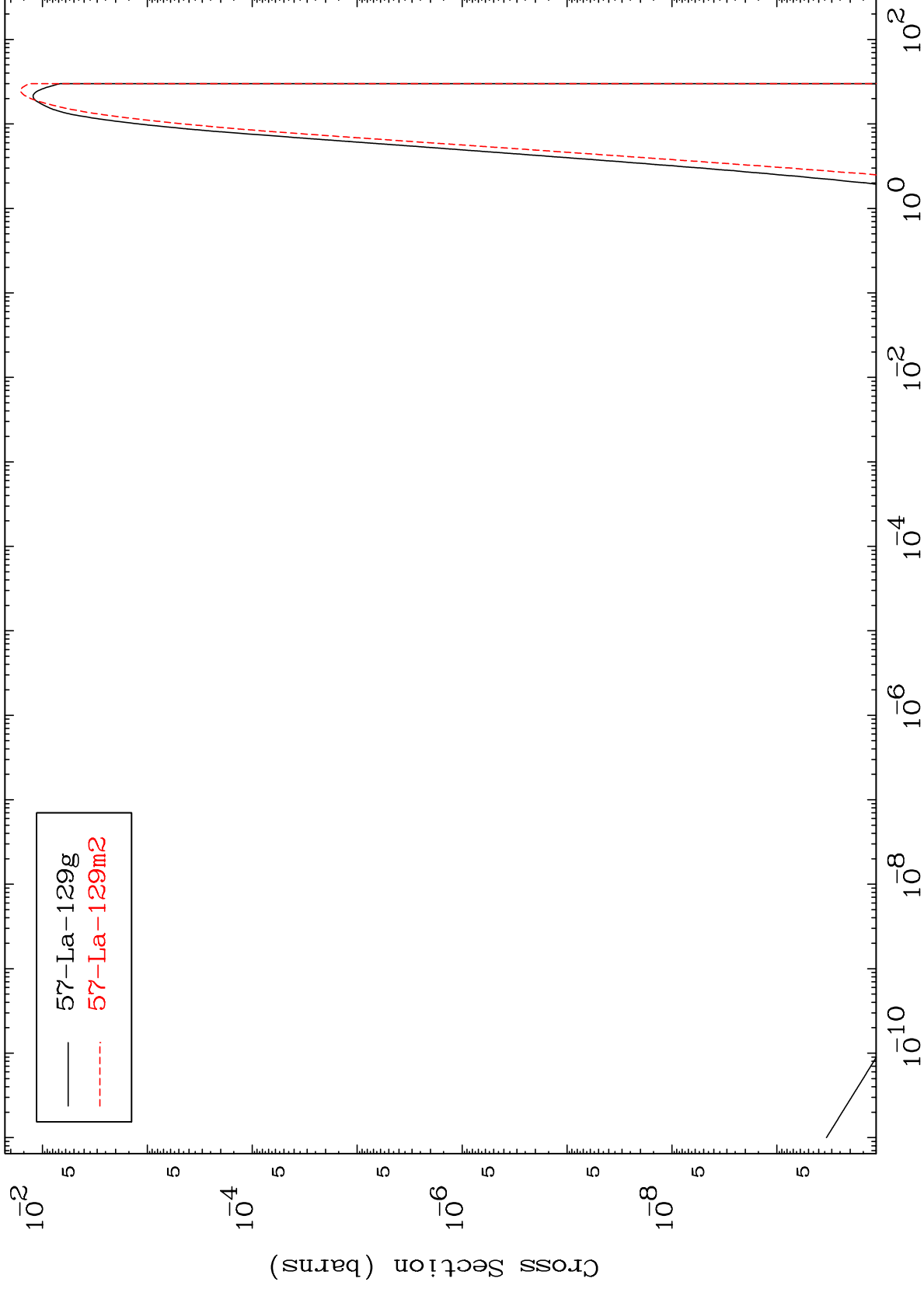


MAT 5999

(n,p) α

60-Nd-133

Radionuclide Production Cross Section



28

Incident Energy (MeV)

60-Nd-133

(n,p) d
Radionuclide Production Cross Section

