

Program EVALPLOT
(Version 2017-1)

by

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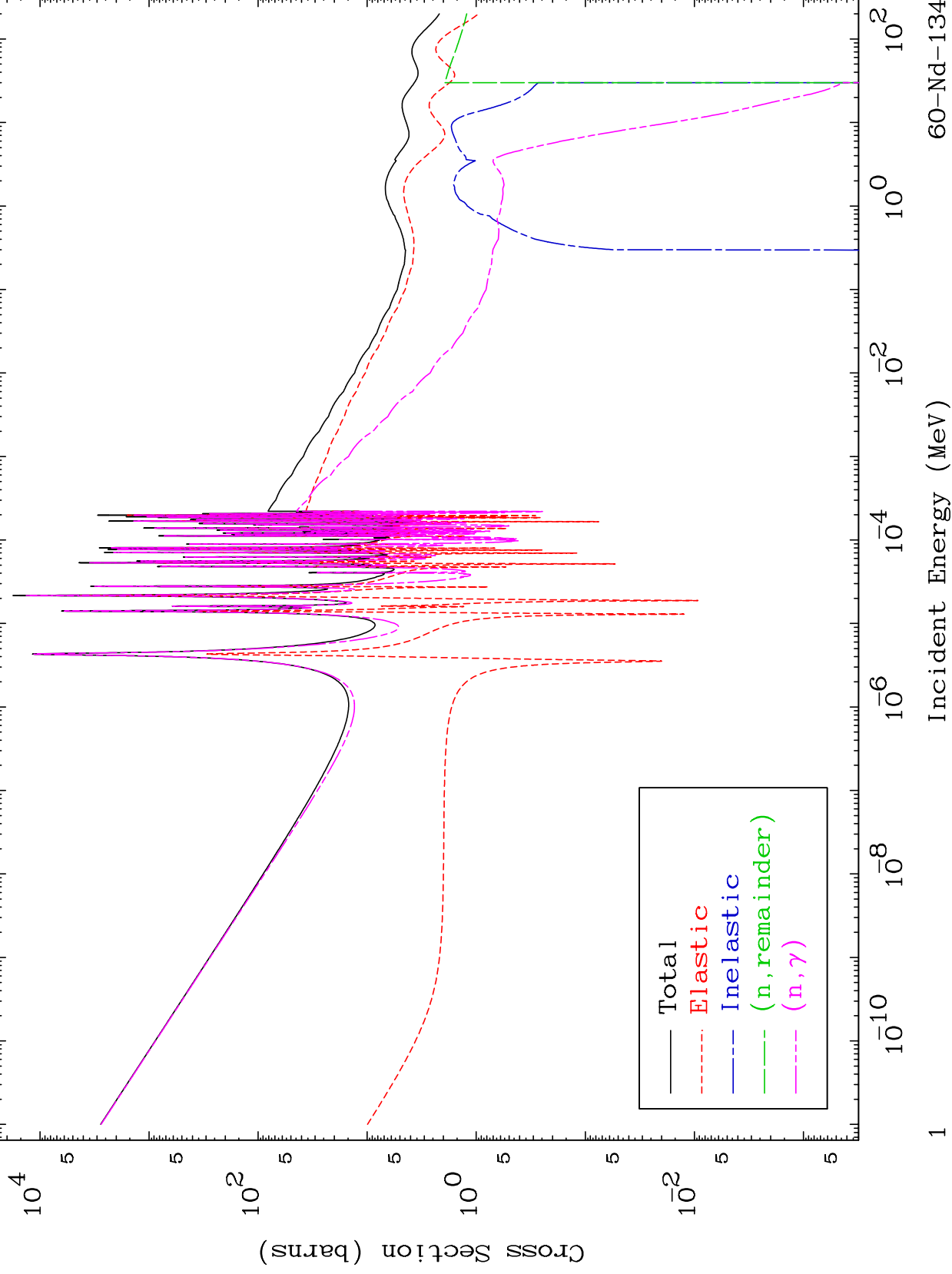
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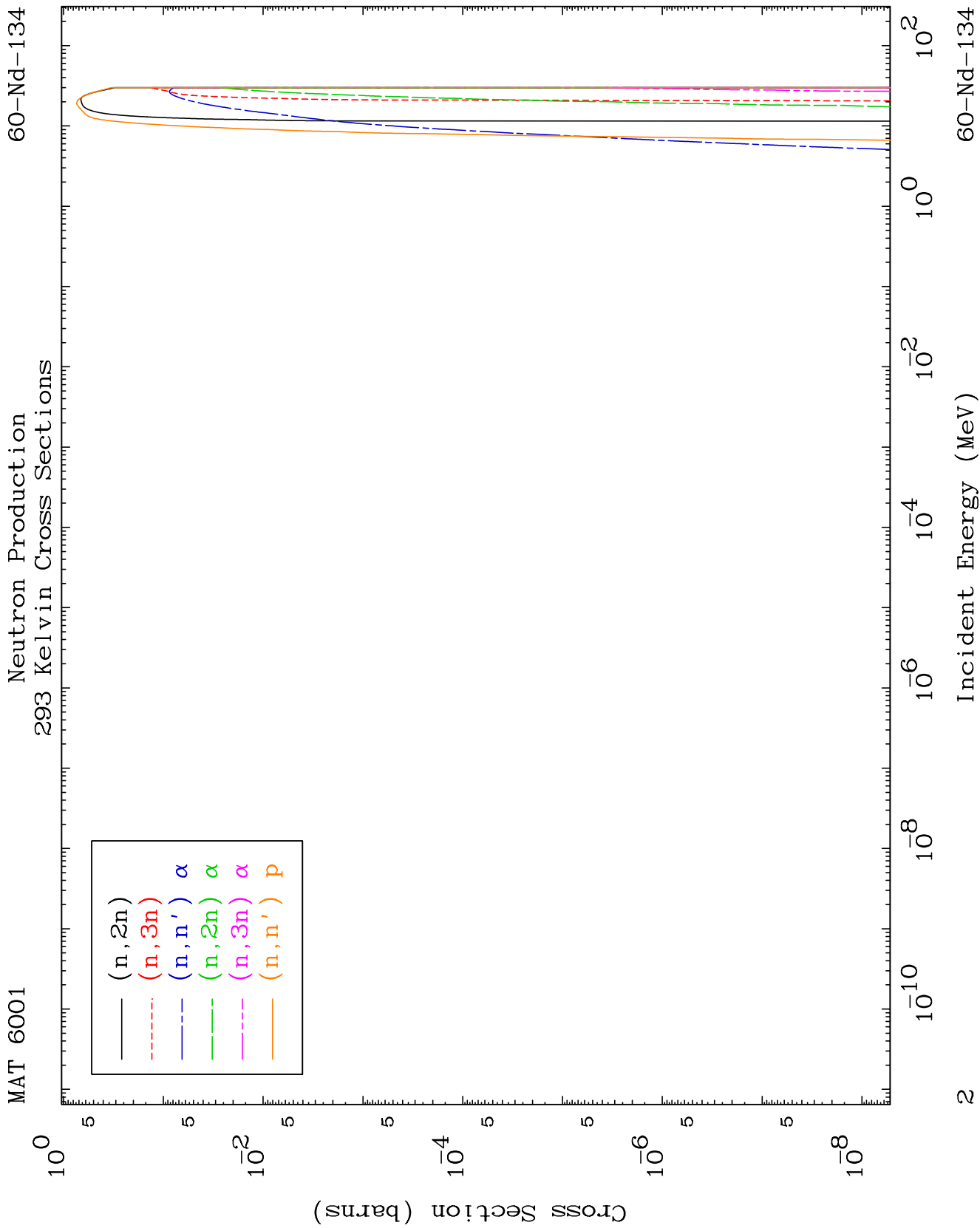
Press Mouse Button to Start

MAT 6001

Major
293 Kelvin Cross Sections

60-Nd-134

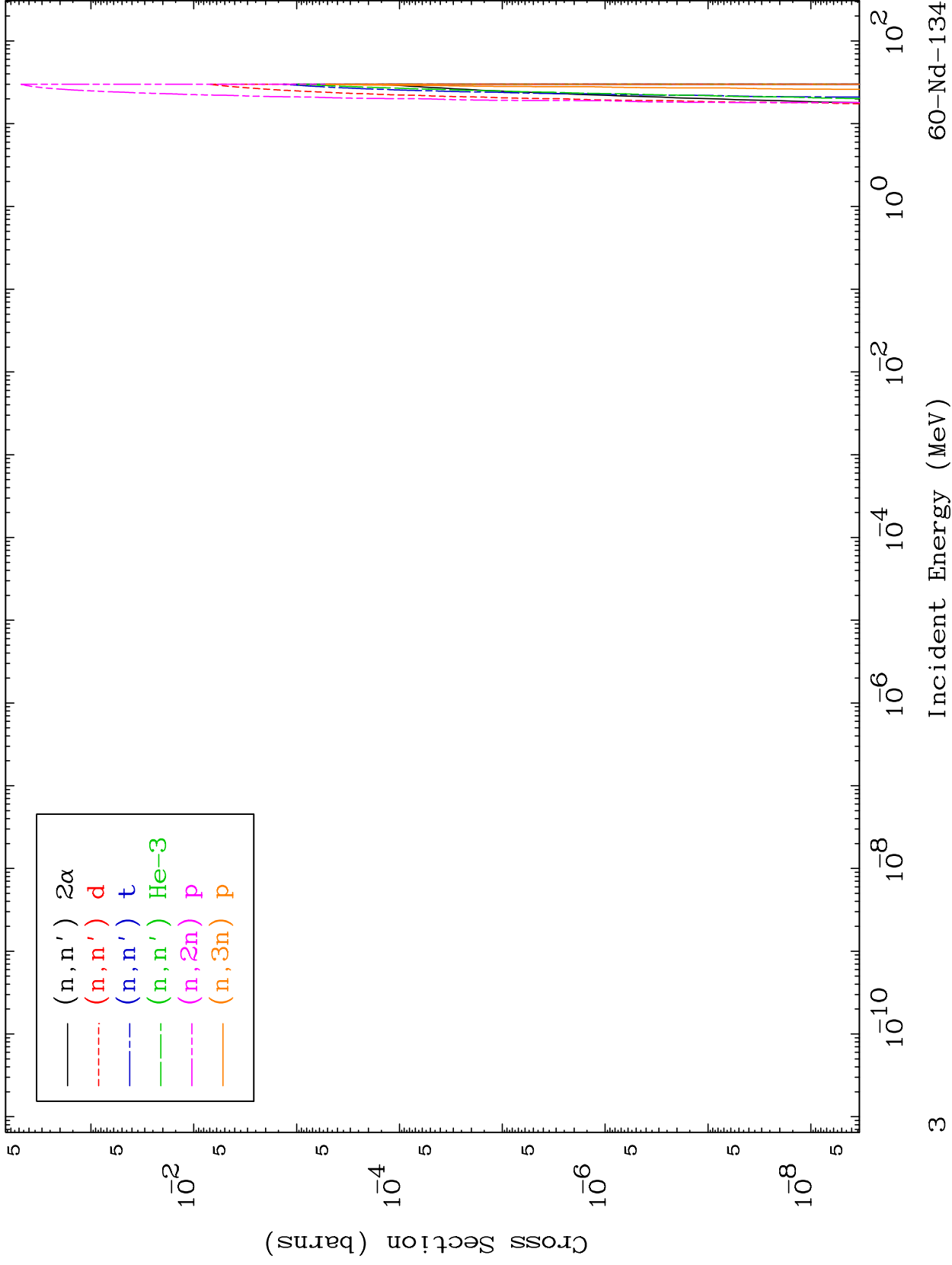


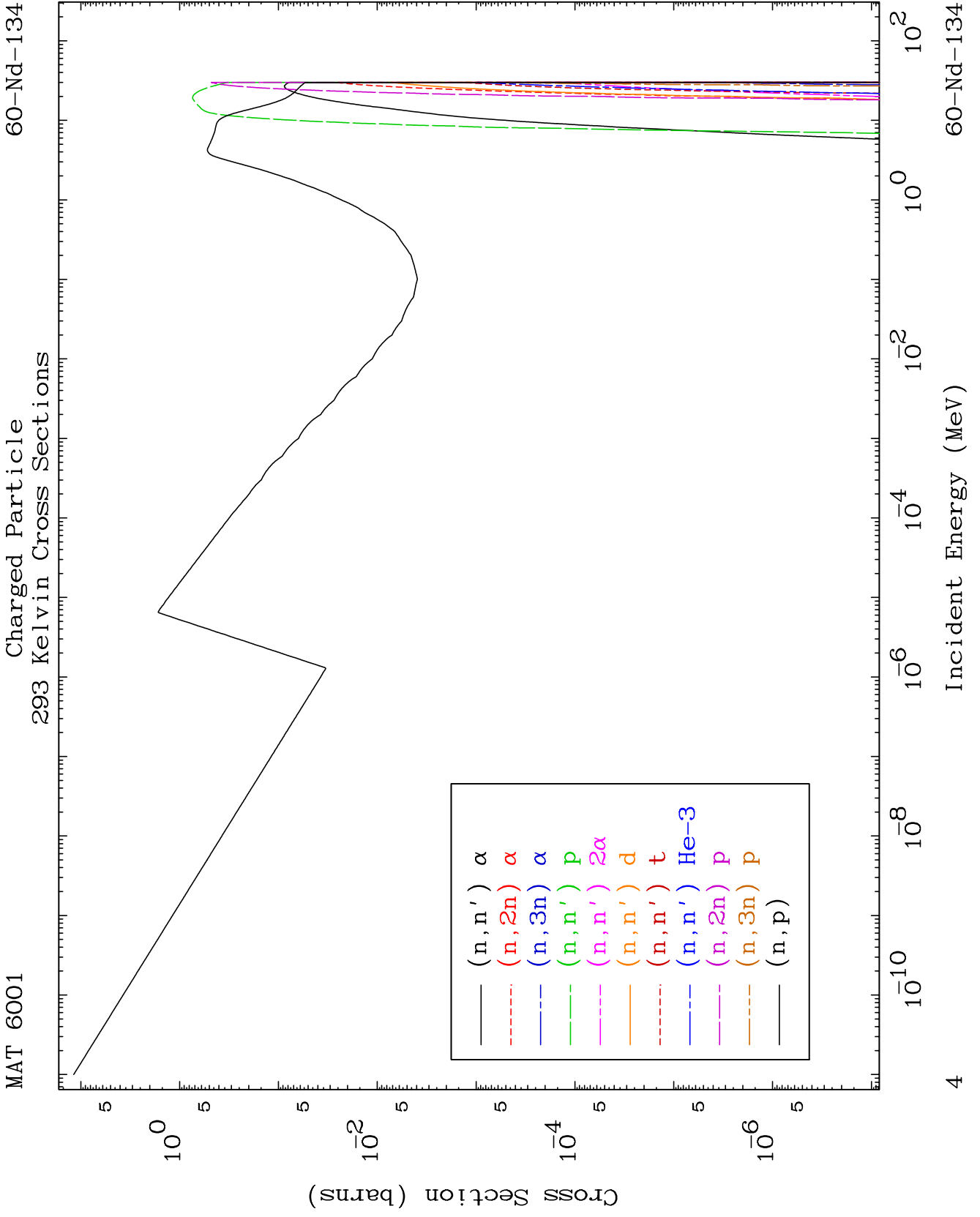


MAT 6001

Neutron Production
293 Kelvin Cross Sections

60-Nd-134

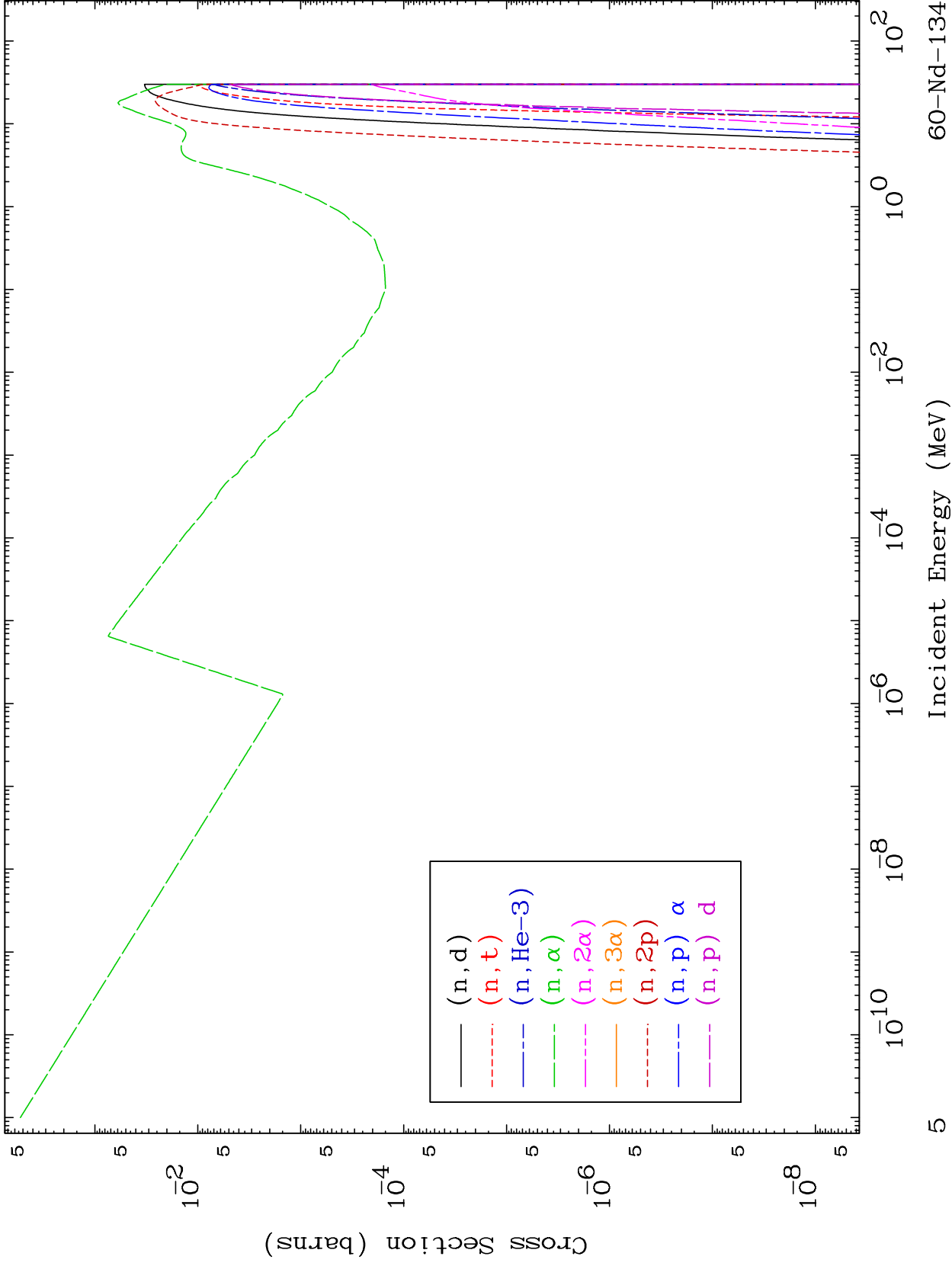




MAT 6001

Charged Particle
293 Kelvin Cross Sections

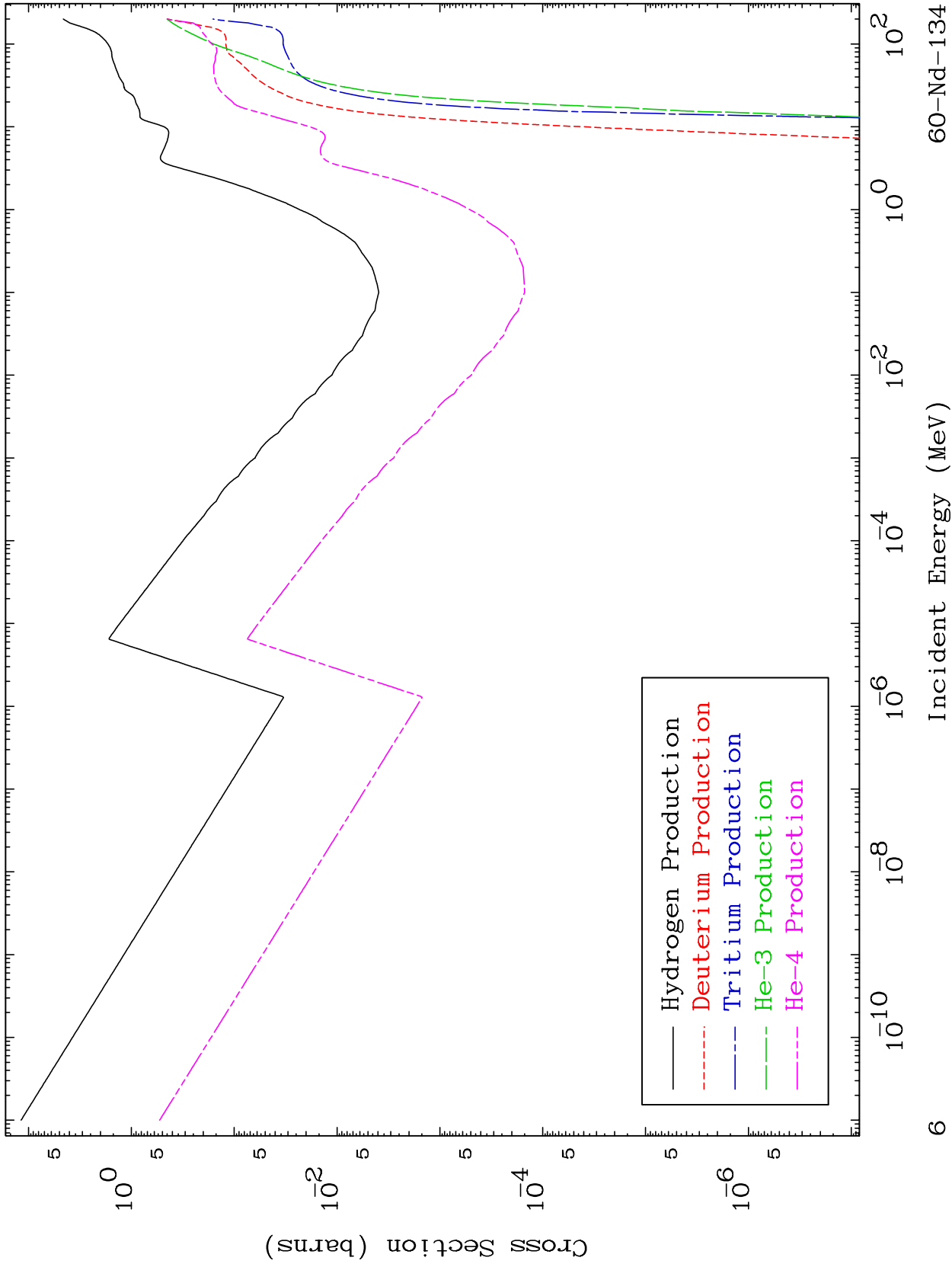
60-Nd-134



MAT 6001

Particle Production
293 Kelvin Cross Sections

60-Nd-134

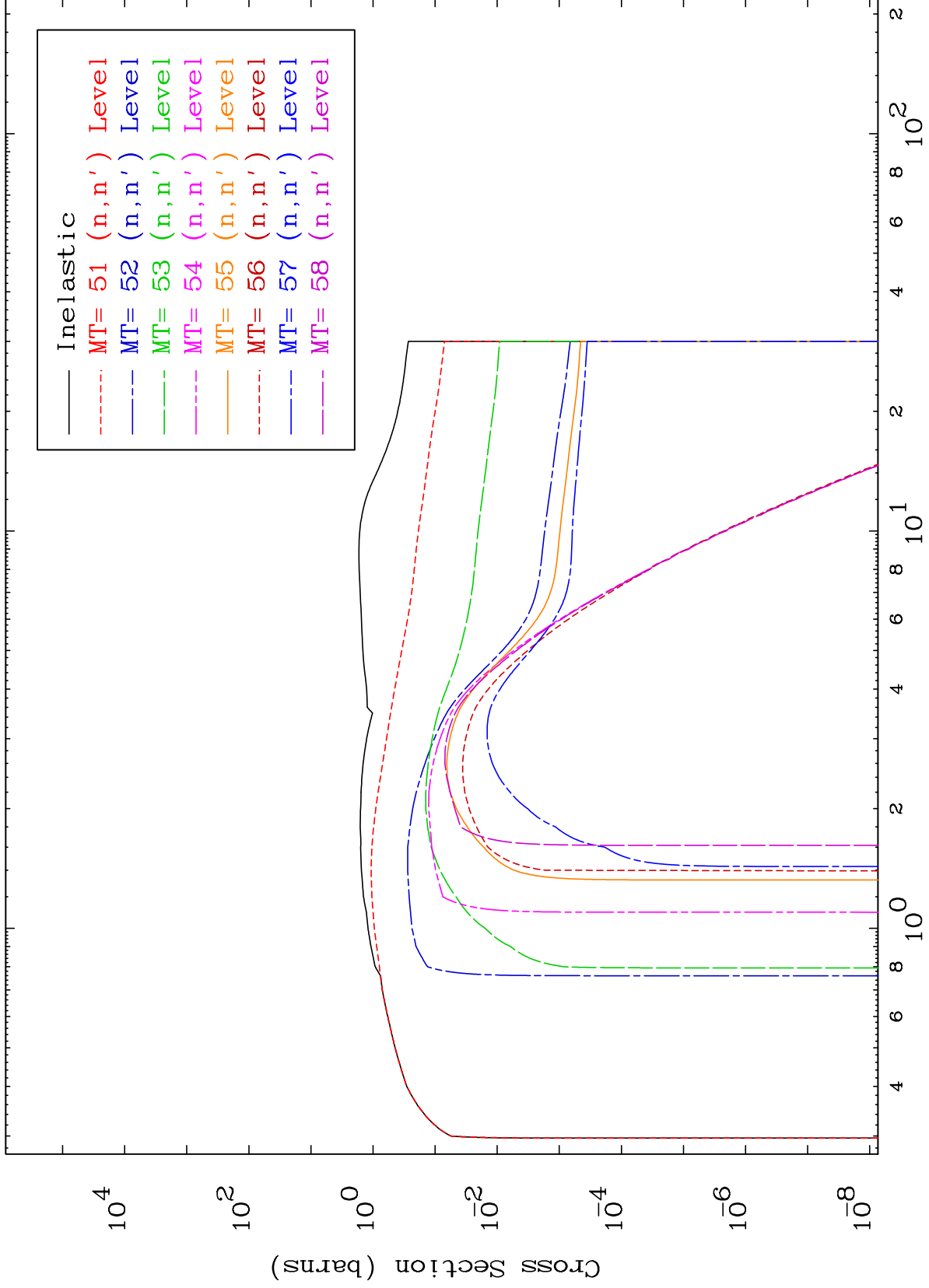


MAT 6001

(n,n') Level

60-Nd-134

293 Kelvin Cross Sections

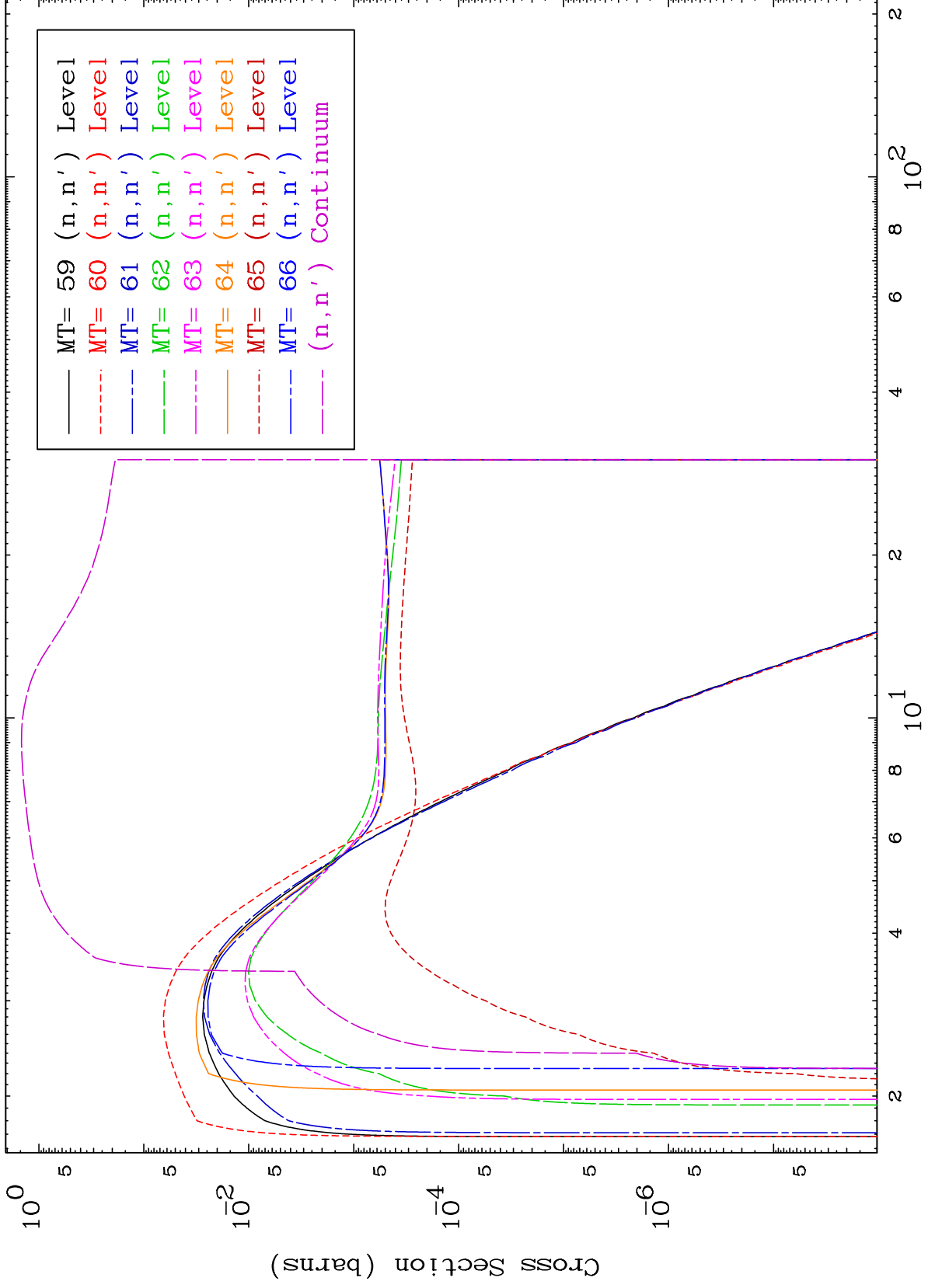


MAT 6001

(n,n') Level

60-Nd-134

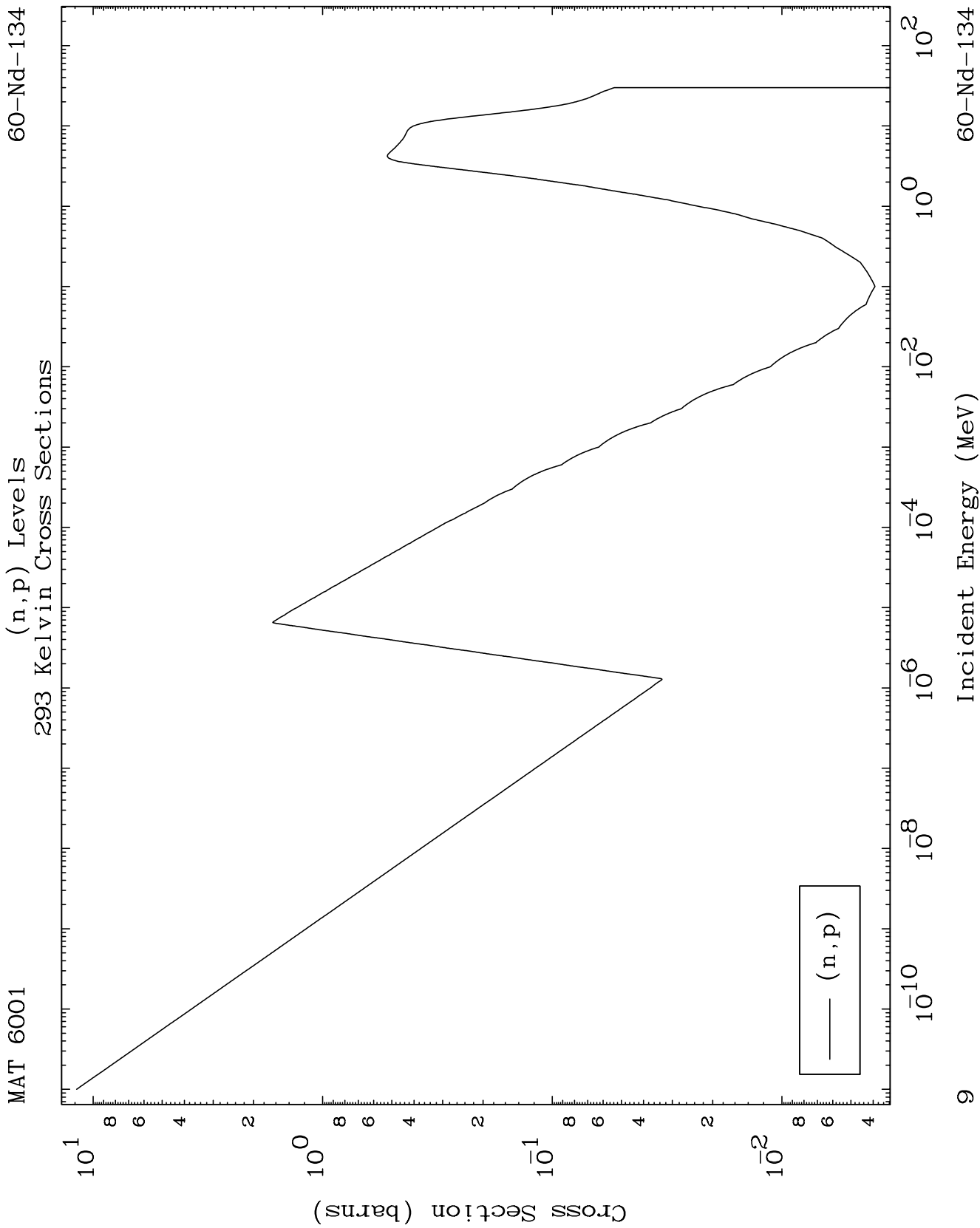
293 Kelvin Cross Sections



8

Incident Energy (MeV)

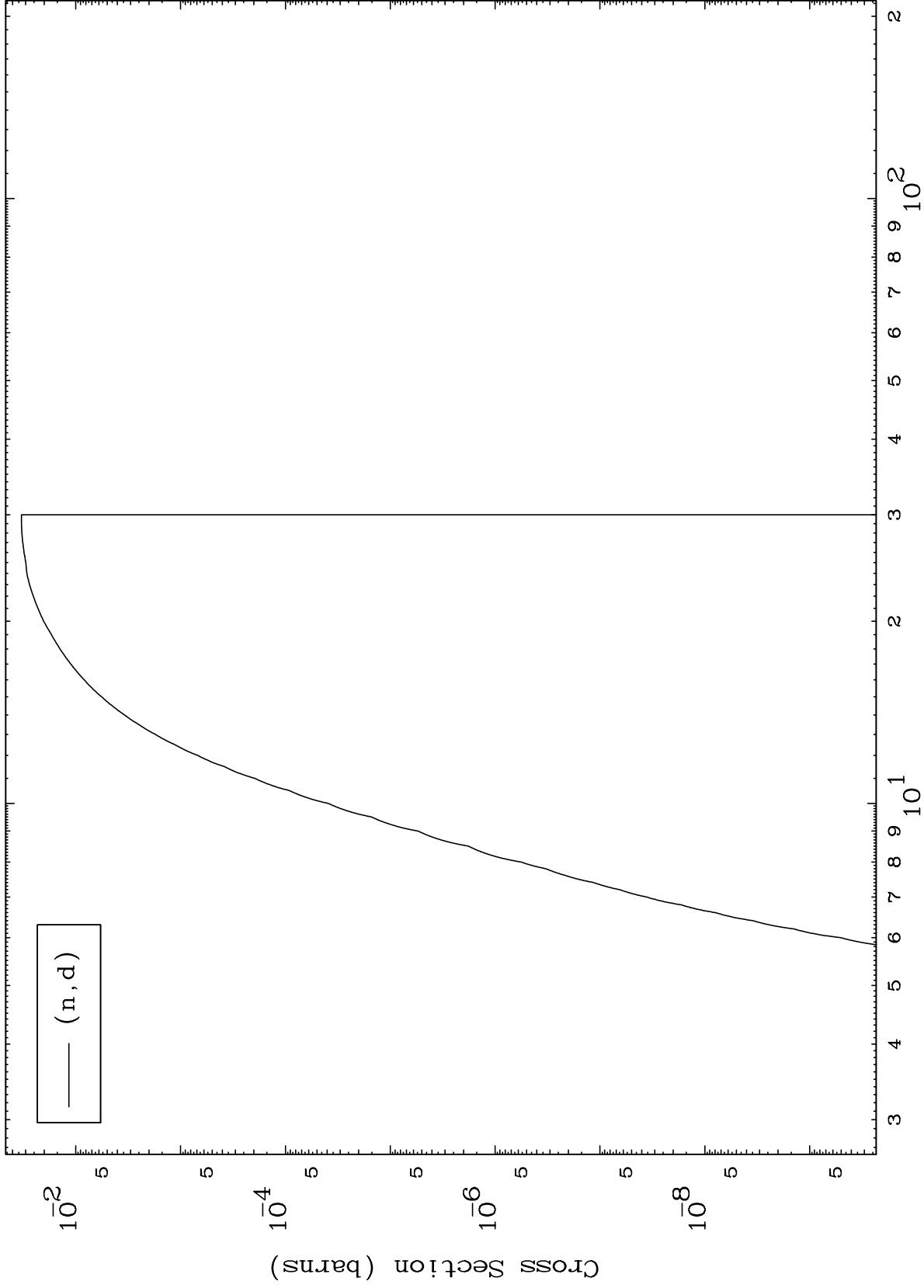
60-Nd-134



MAT 6001

(n,d) Levels
293 Kelvin Cross Sections

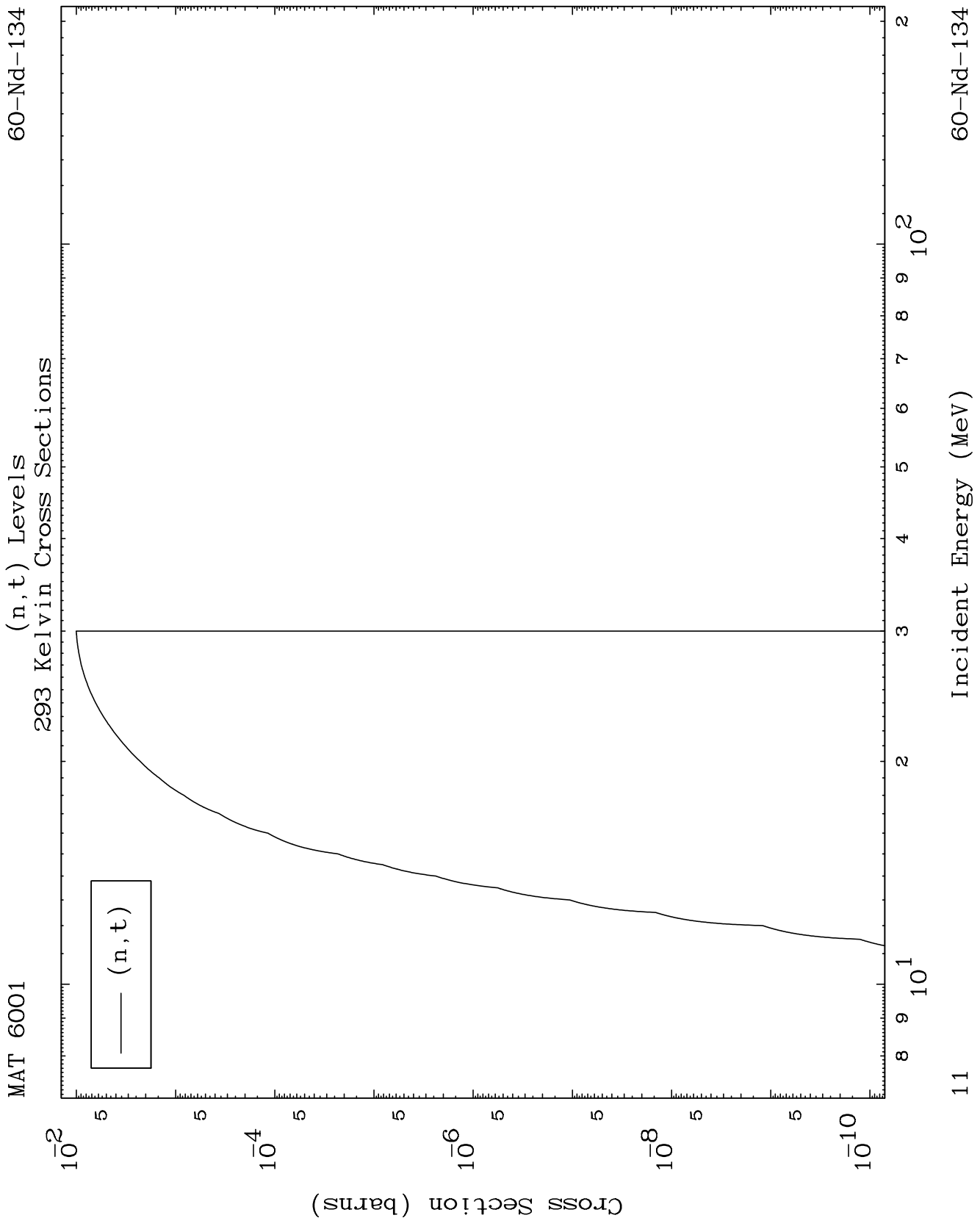
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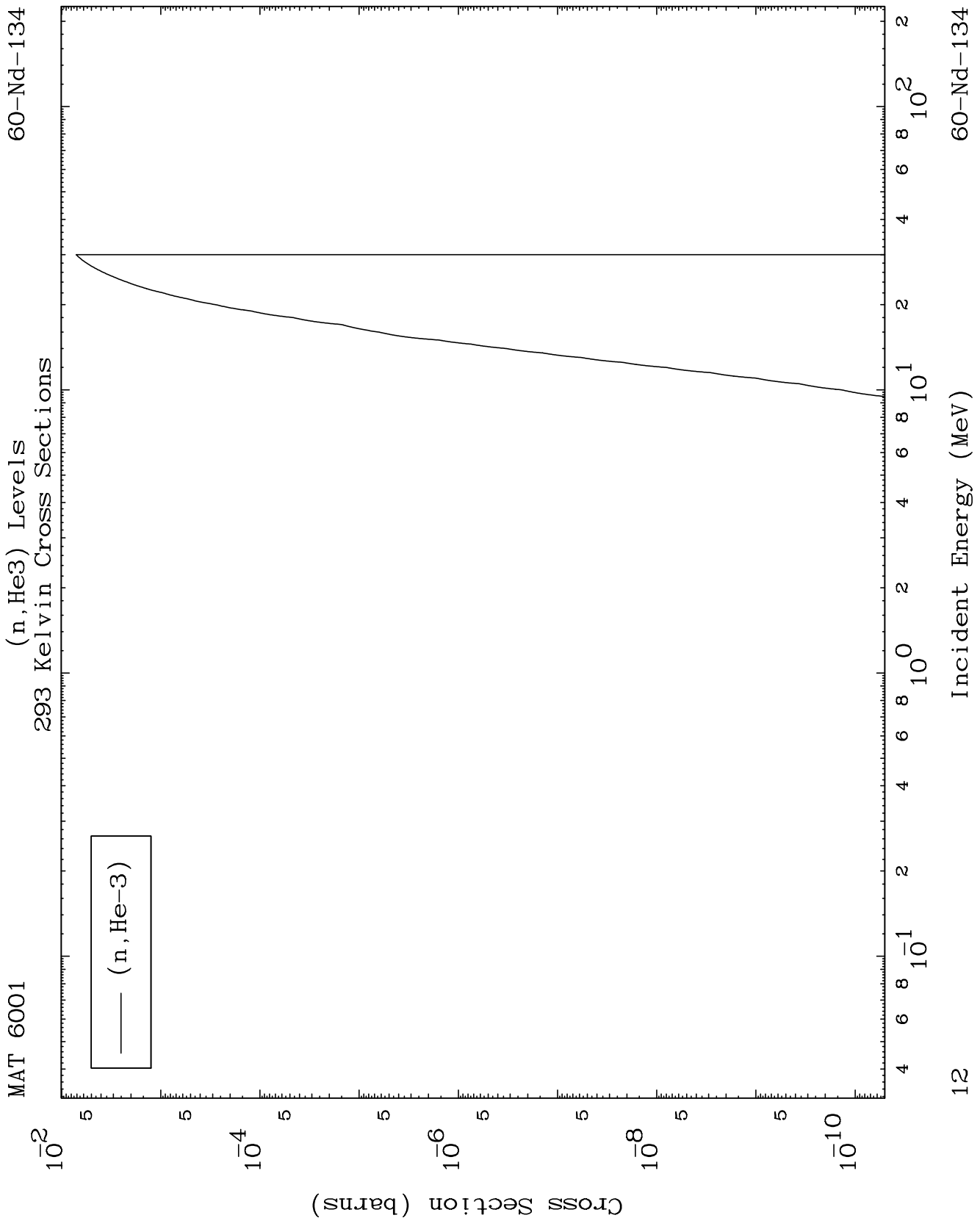


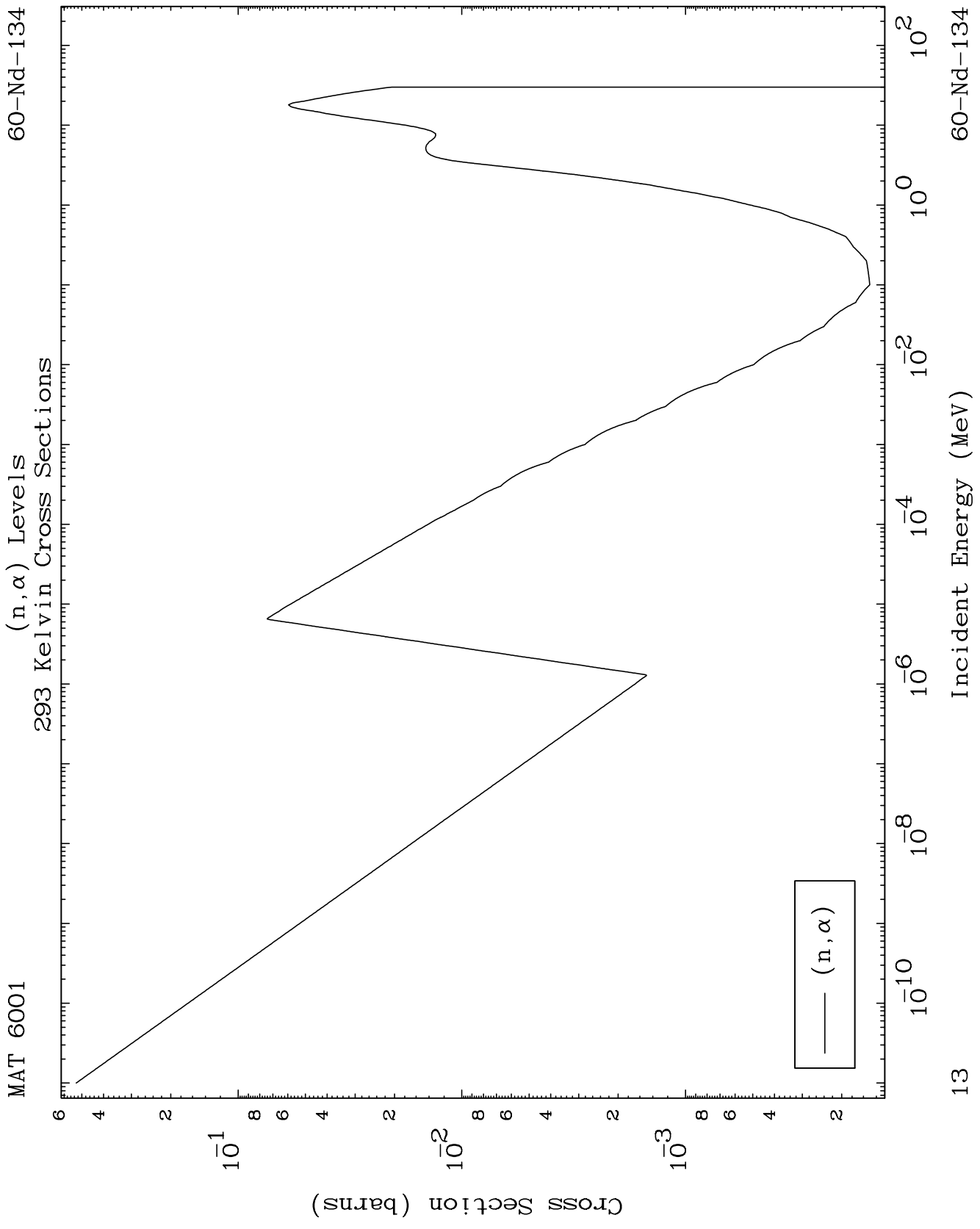
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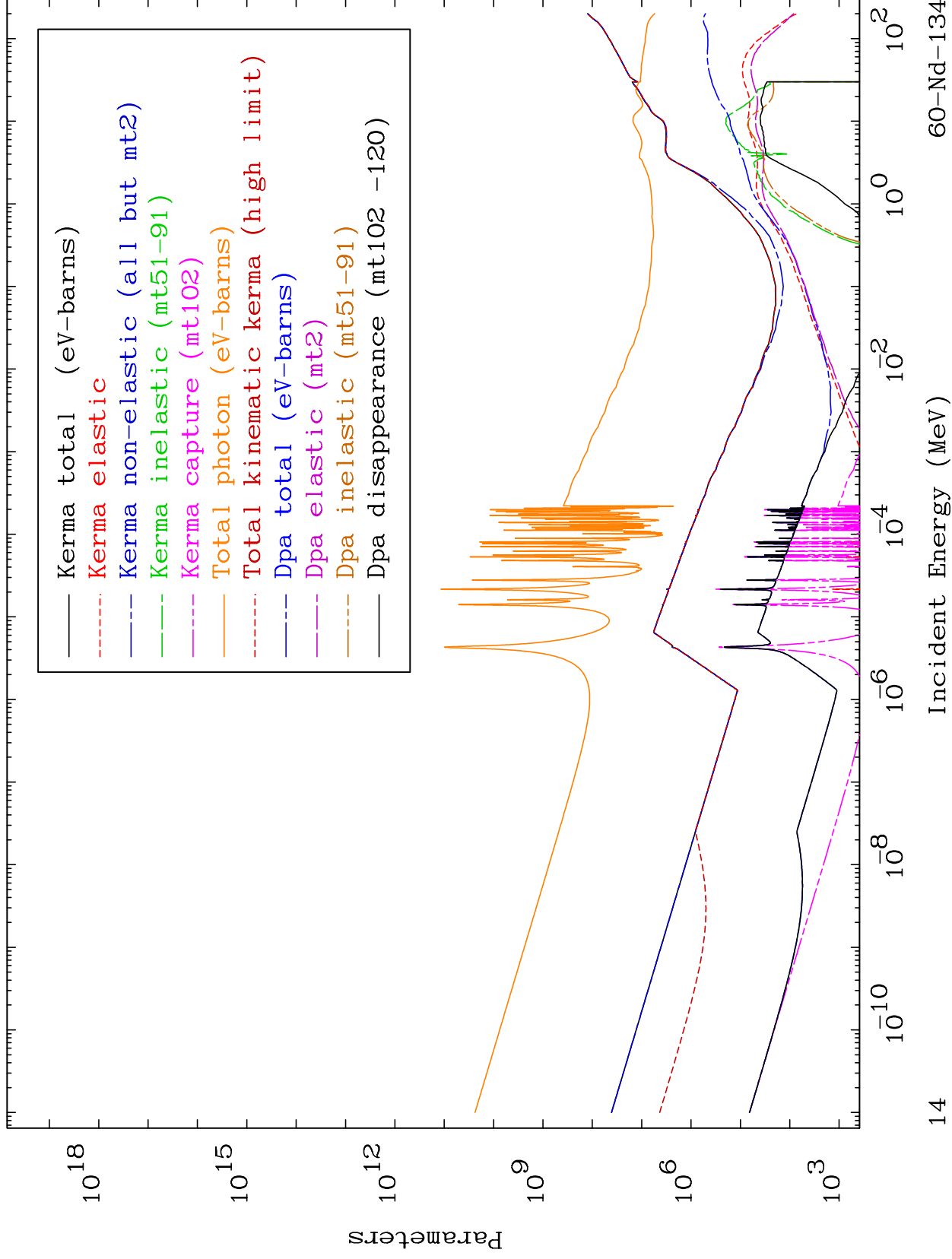
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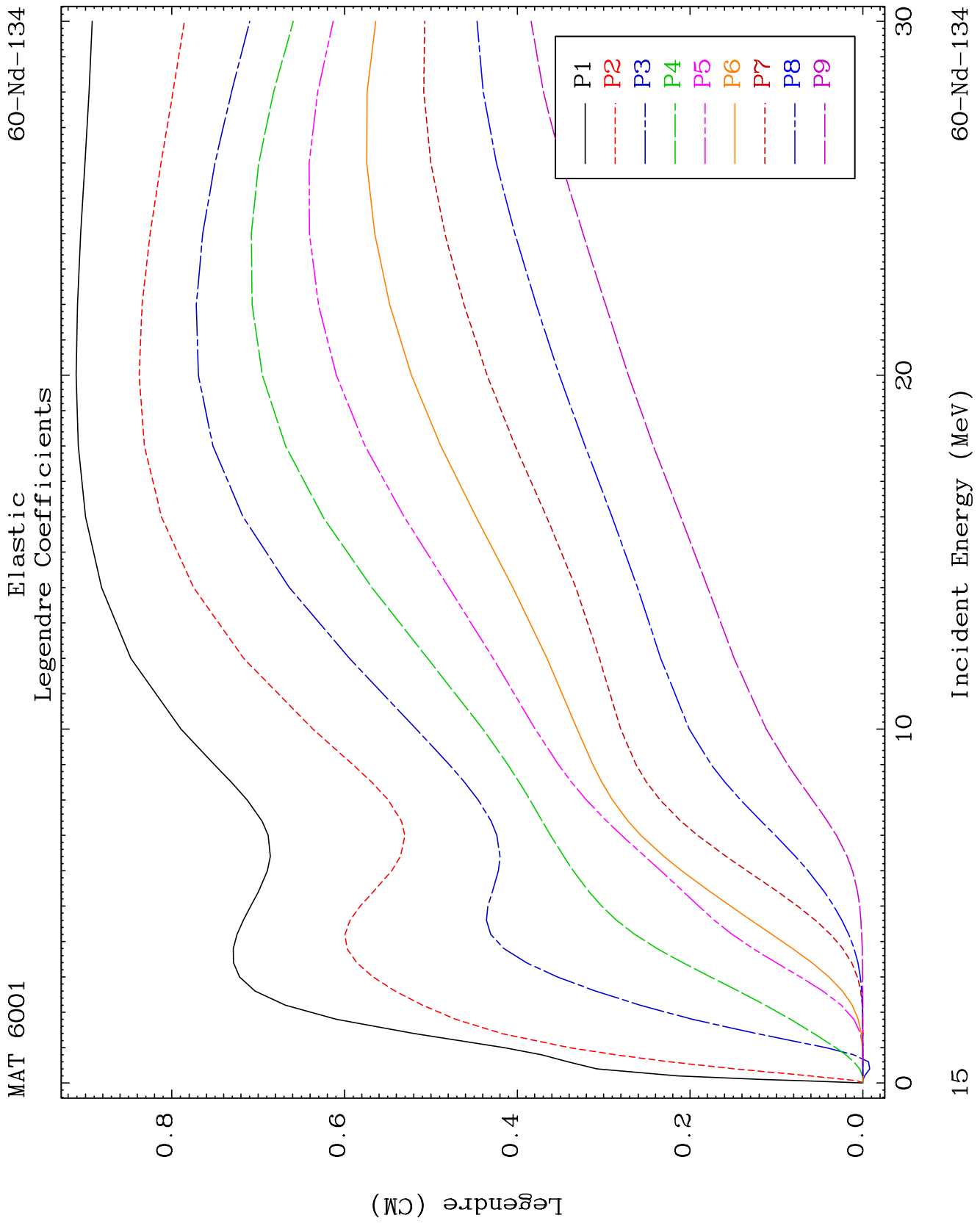
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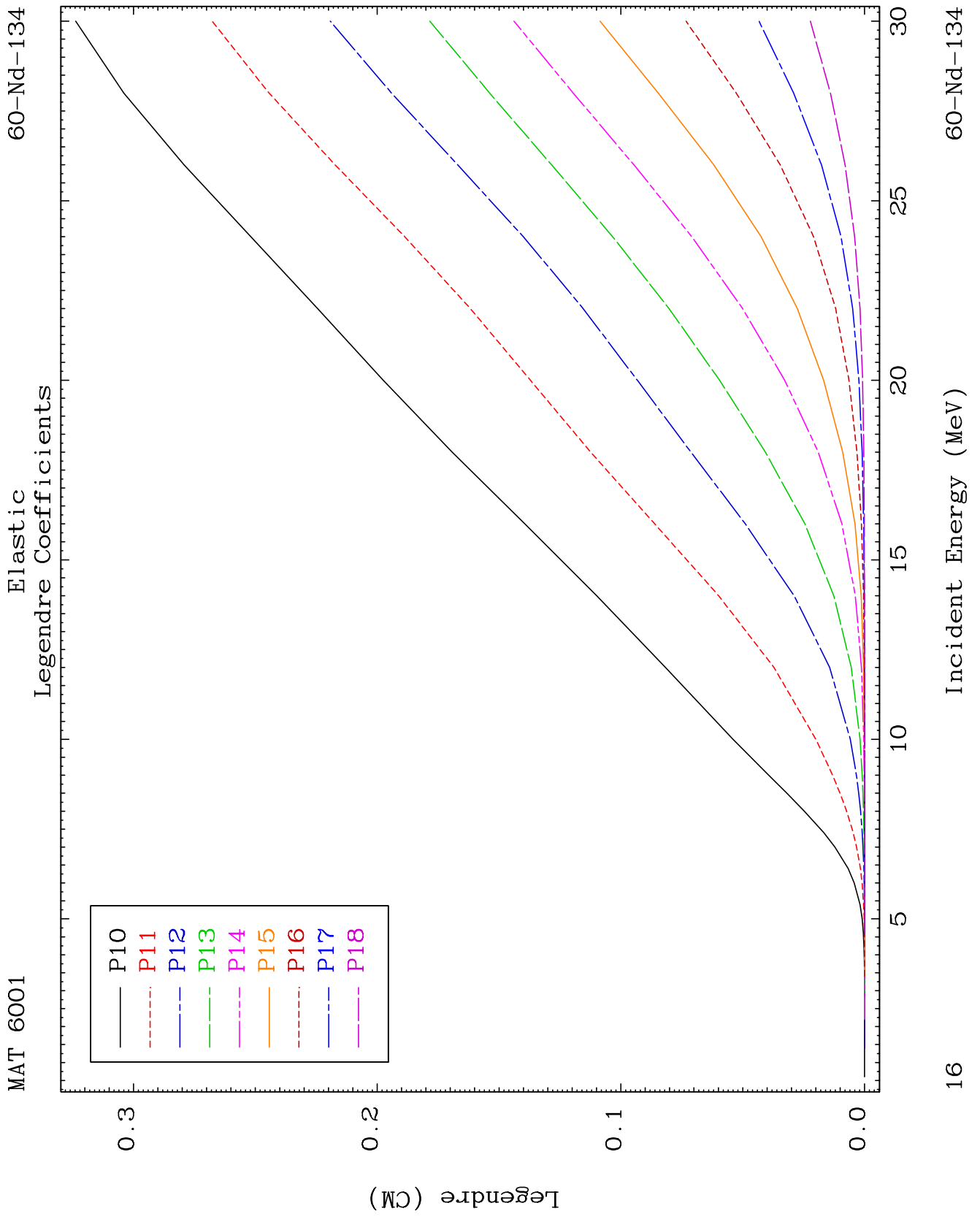


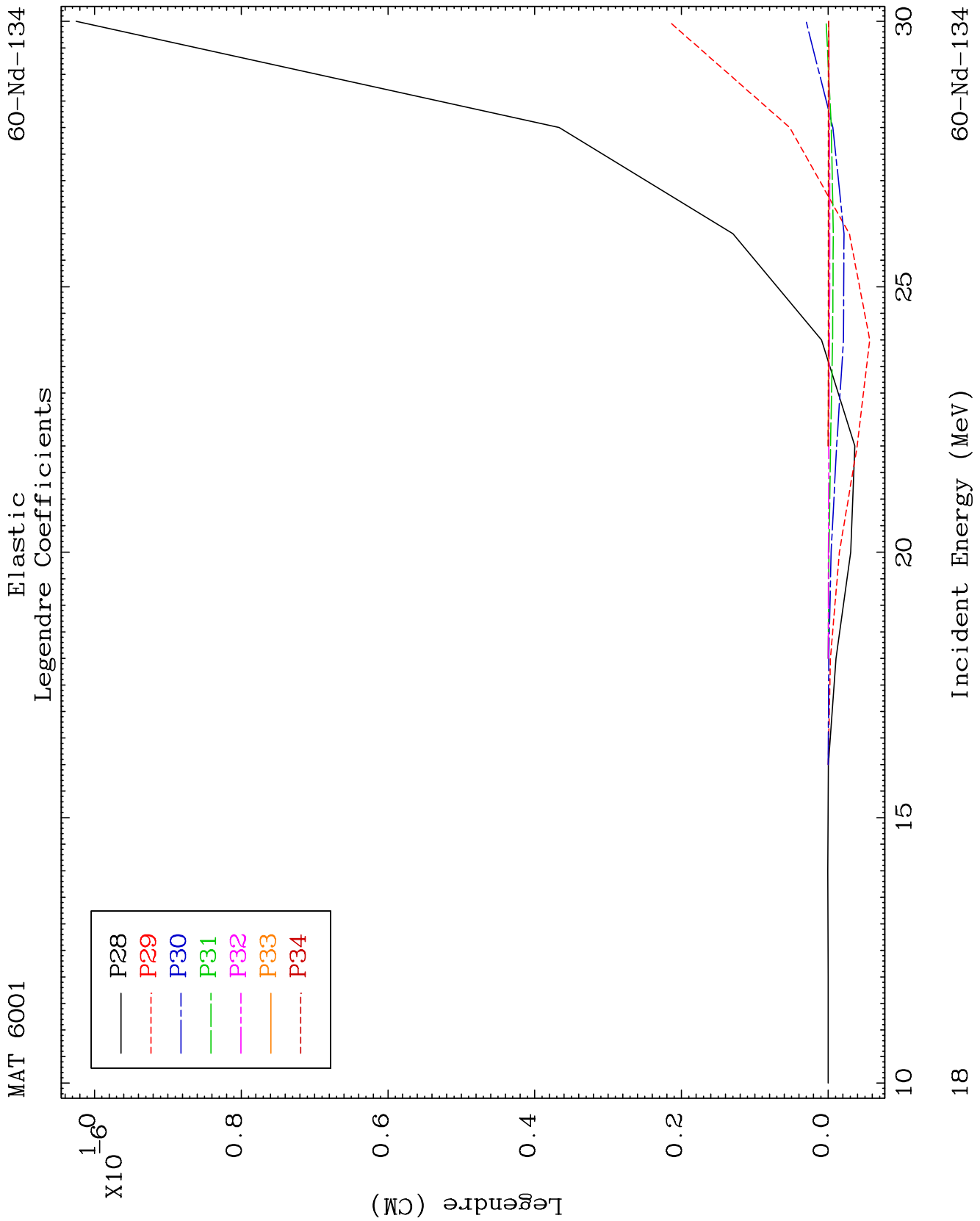


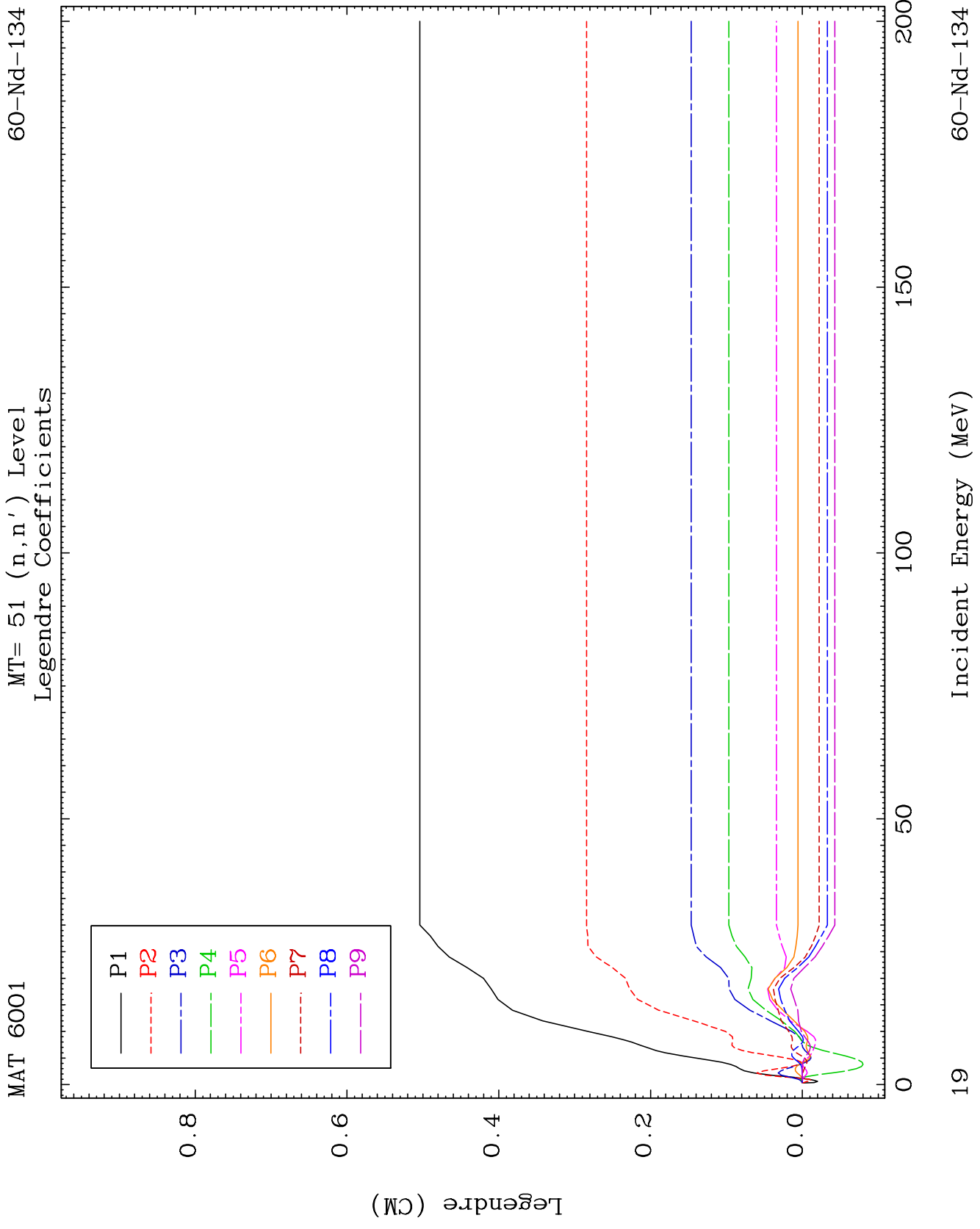


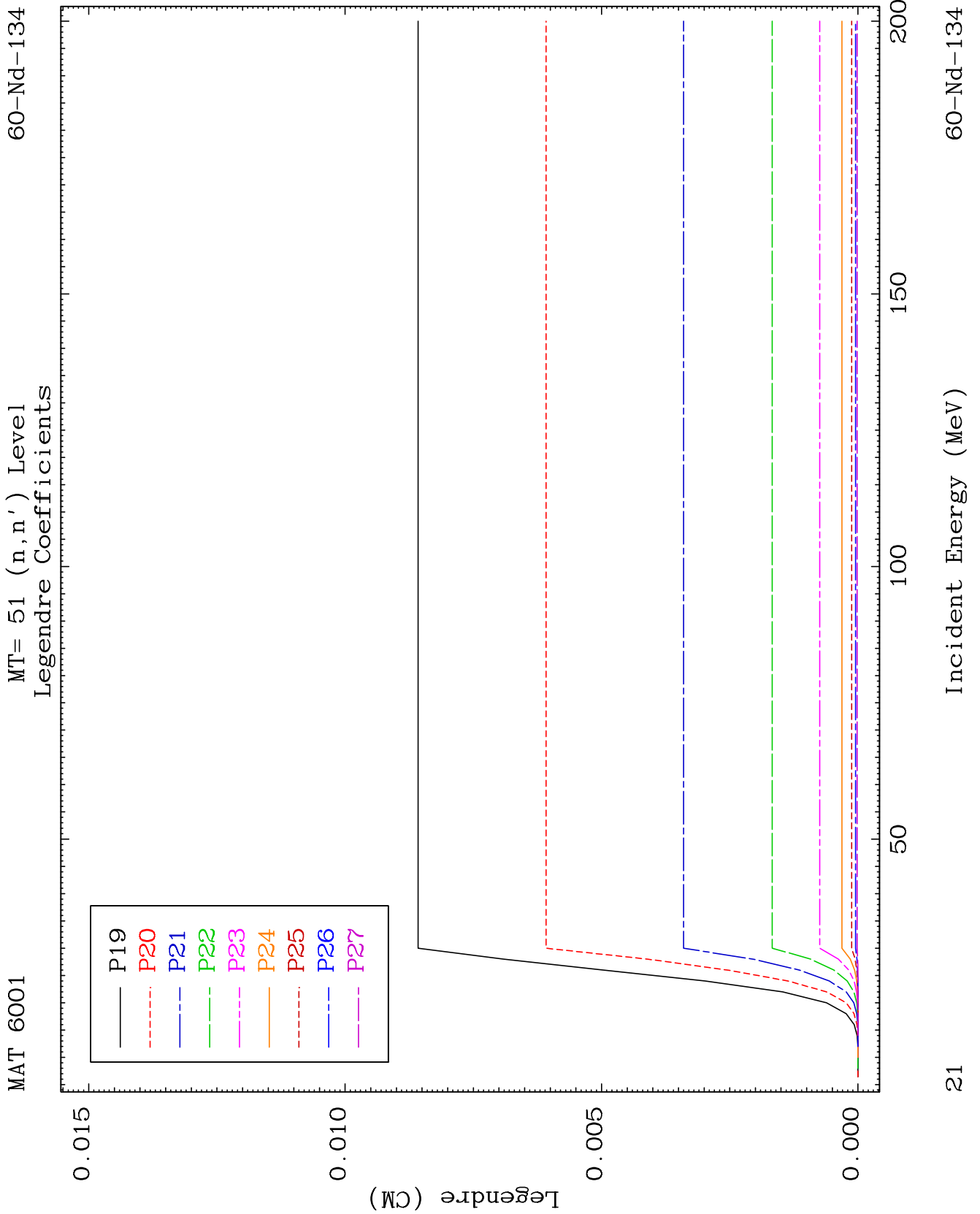


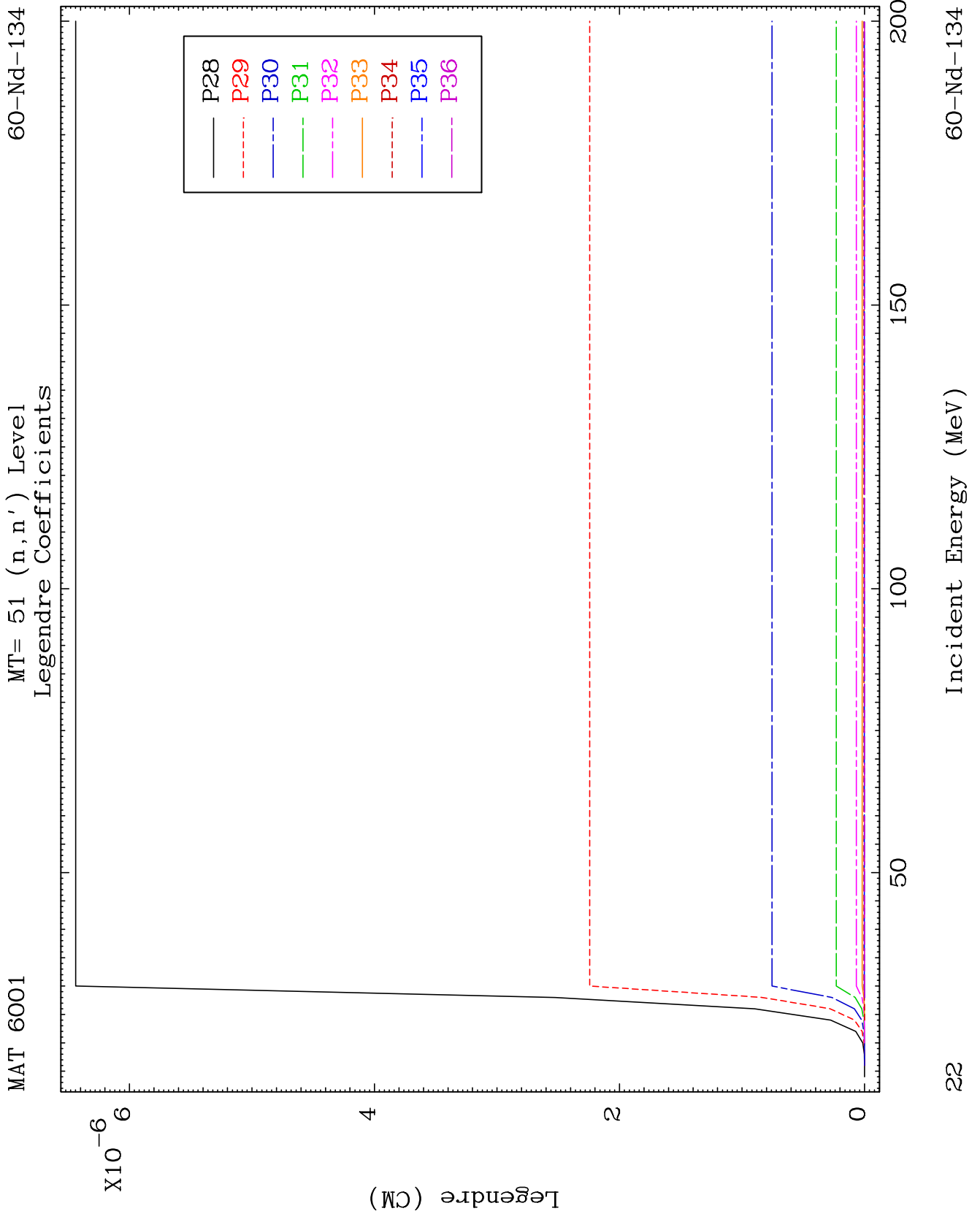


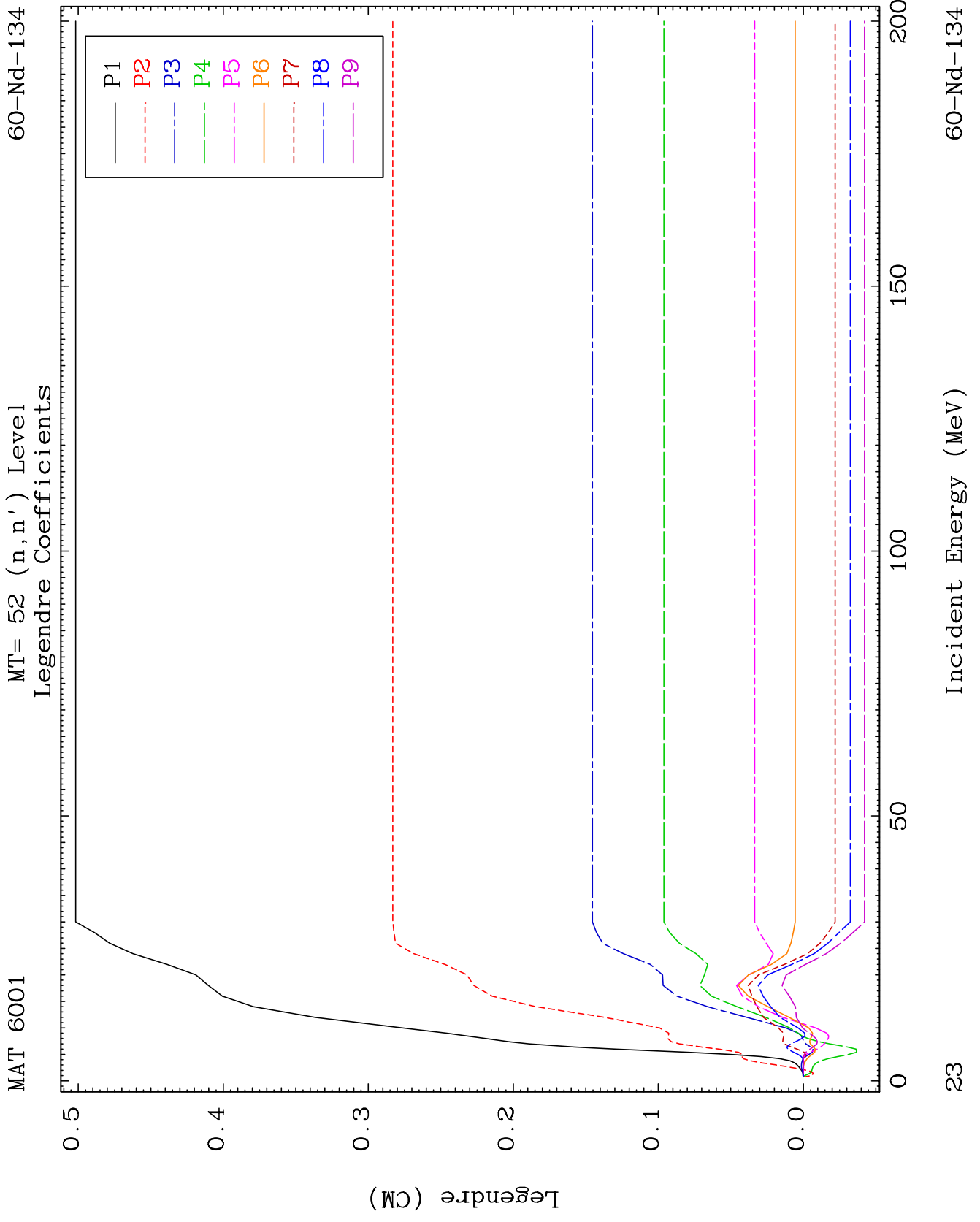


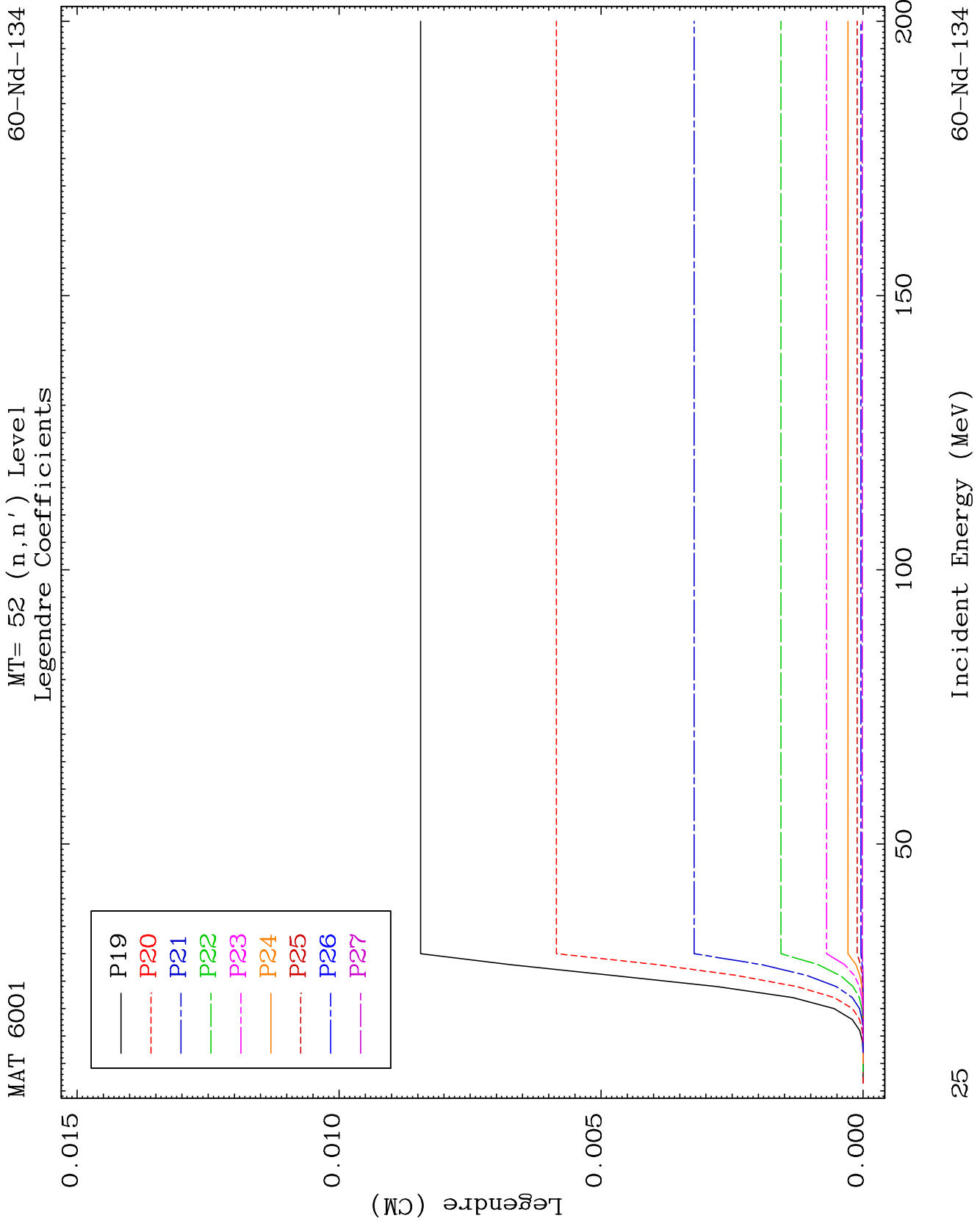


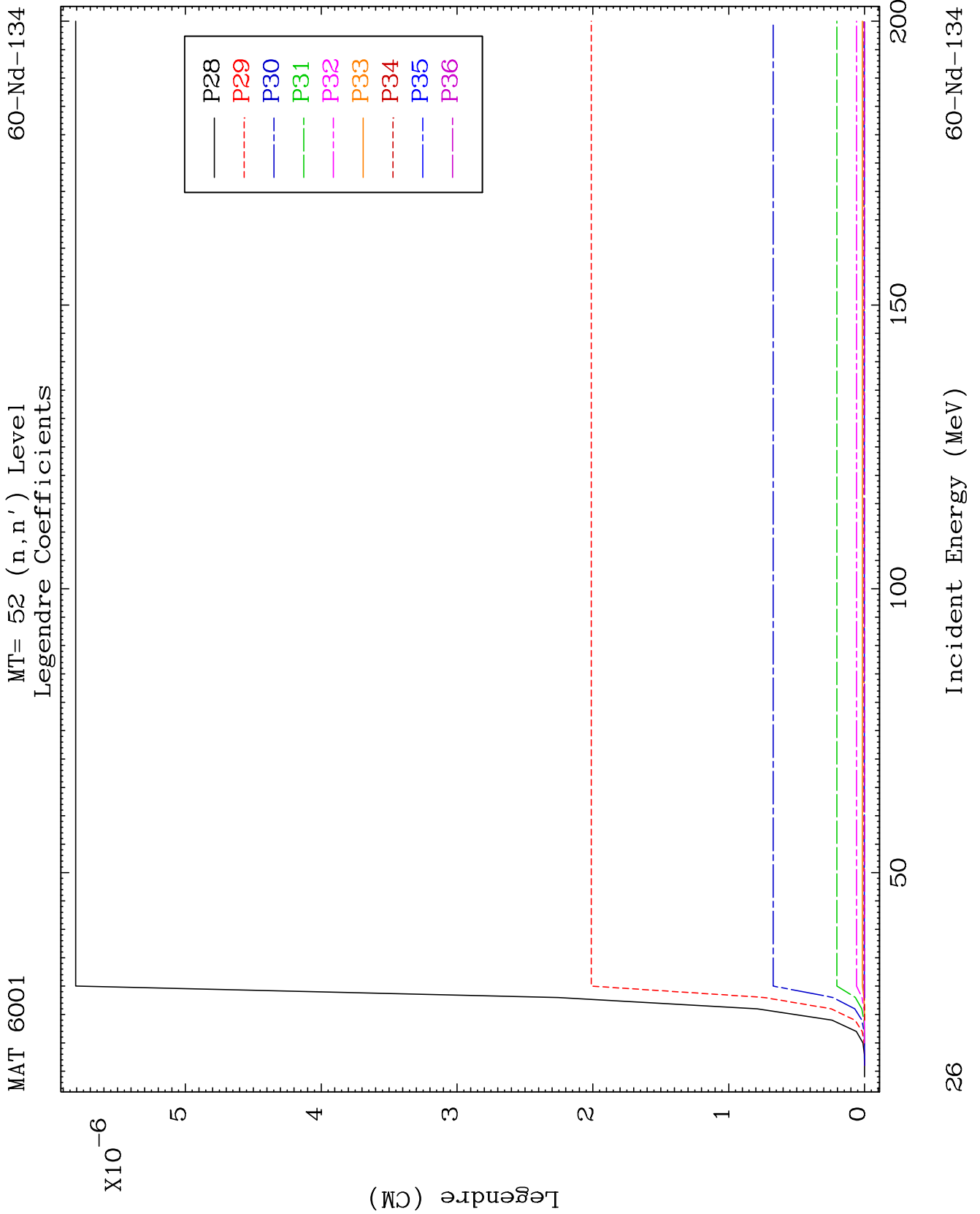


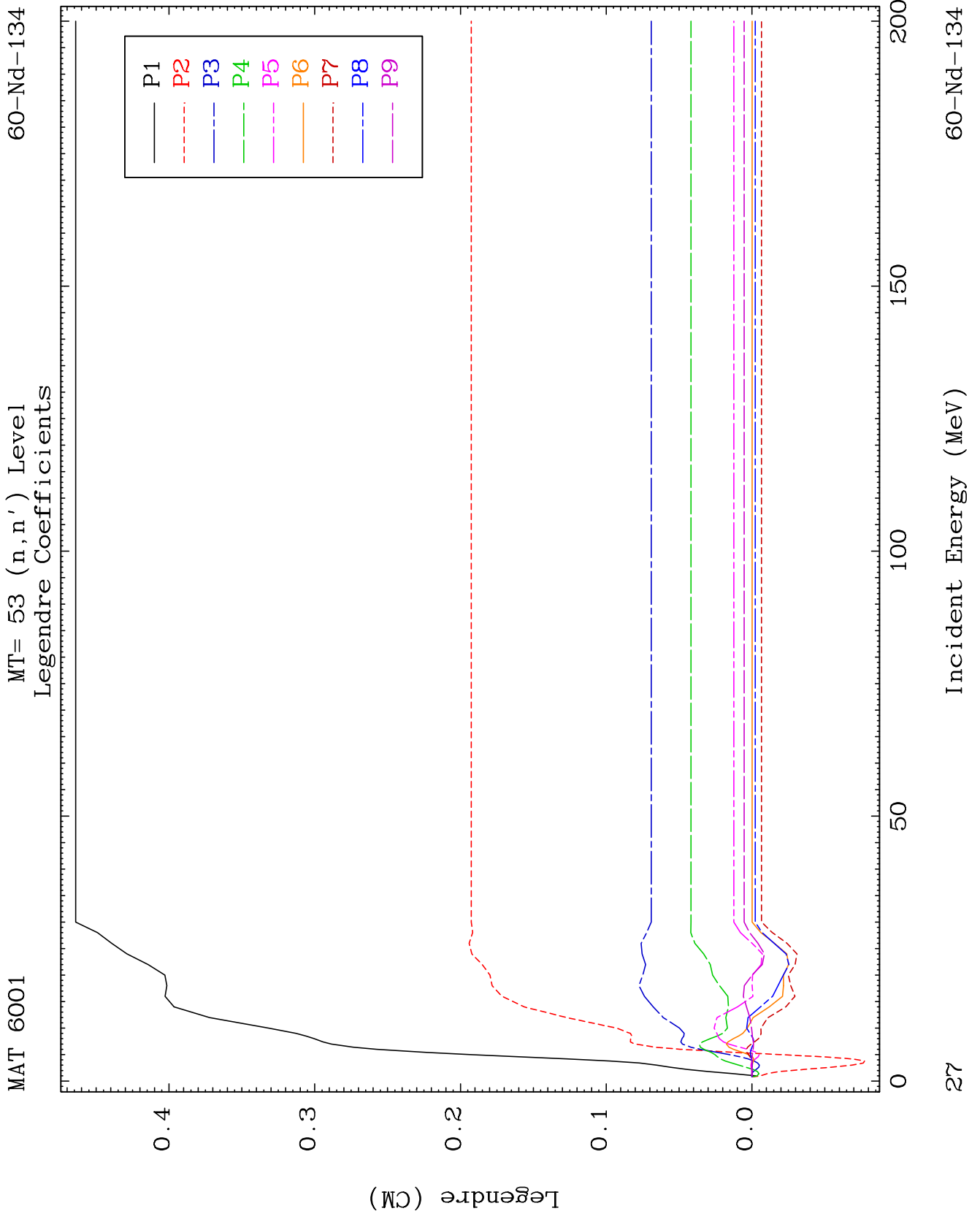


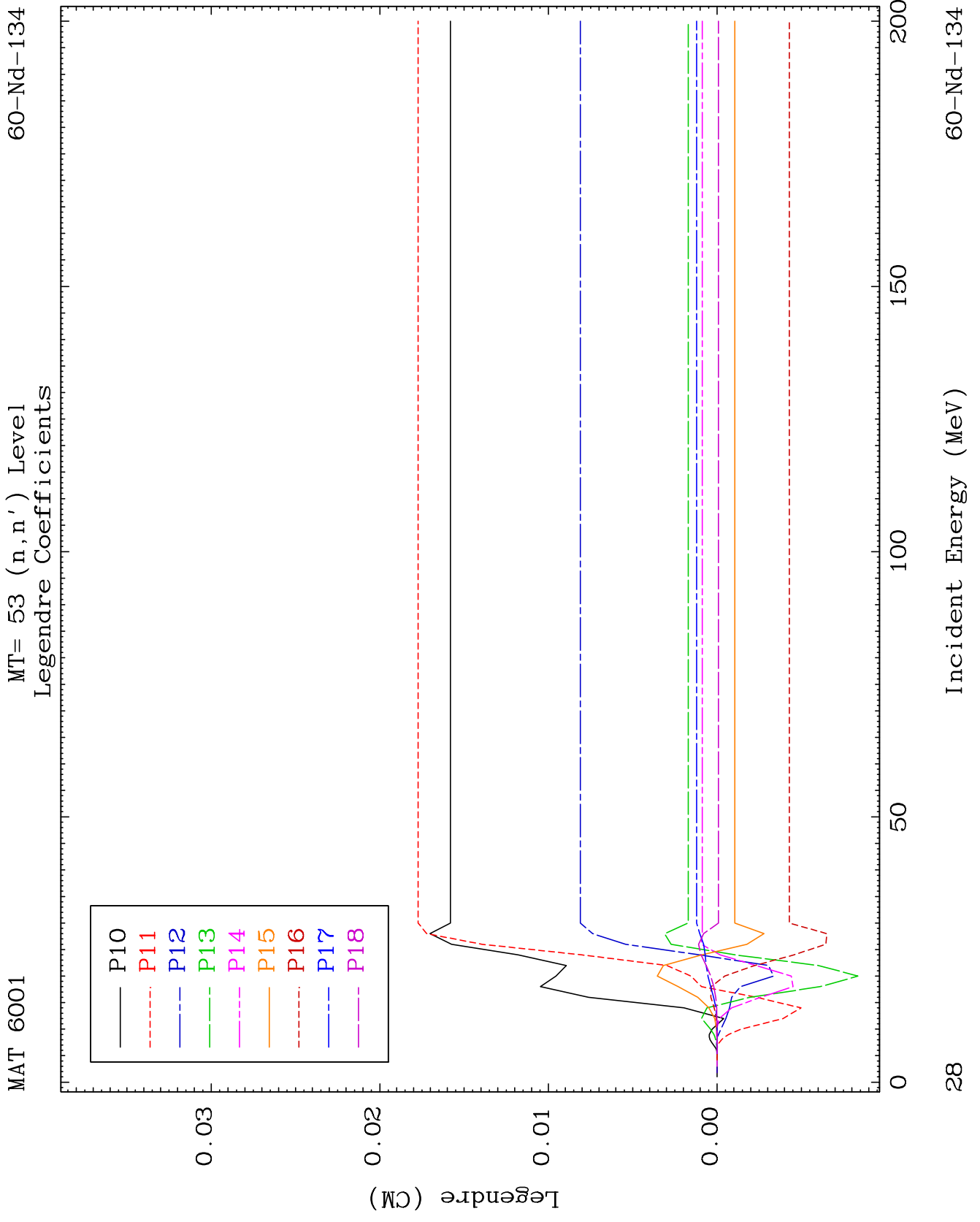


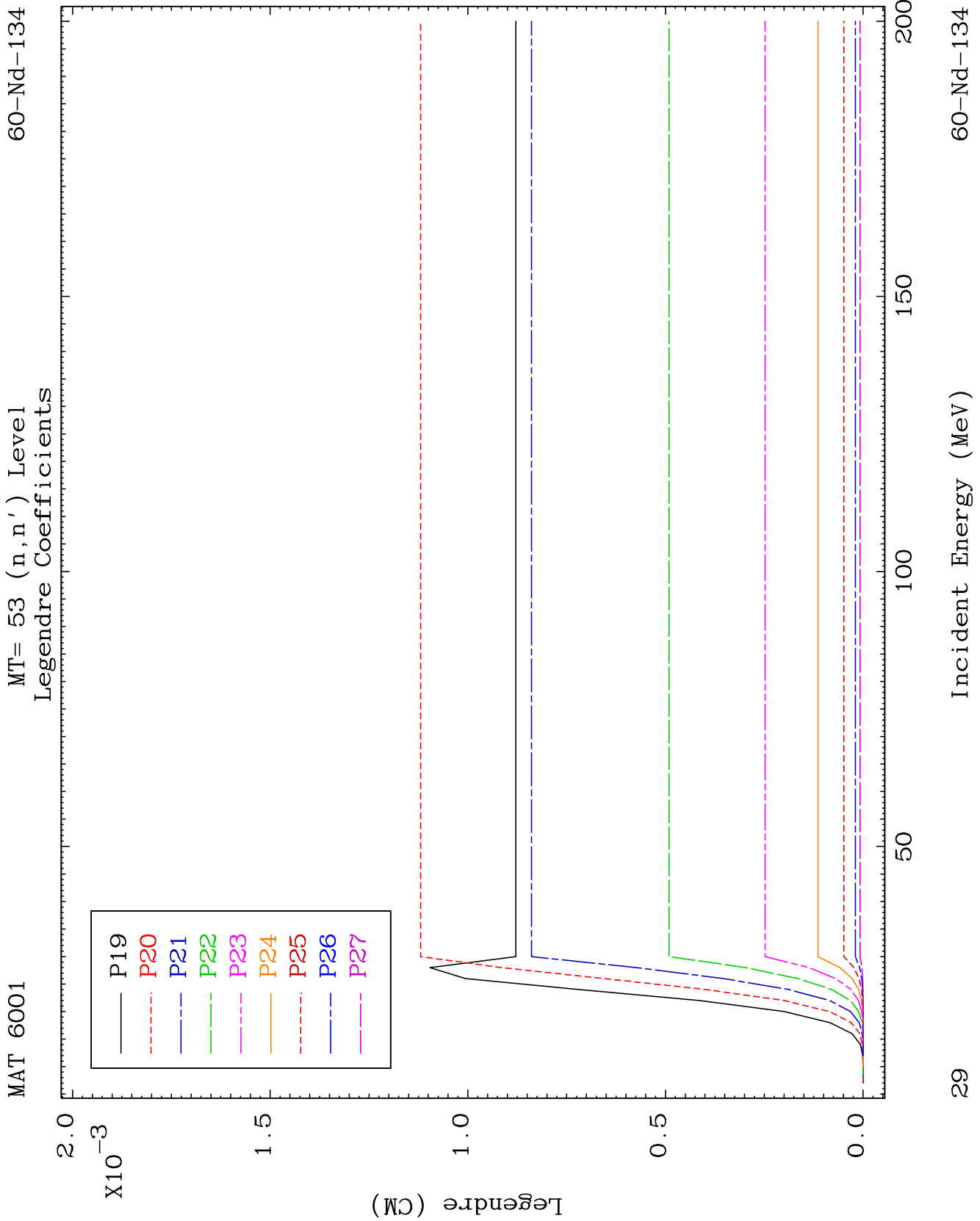


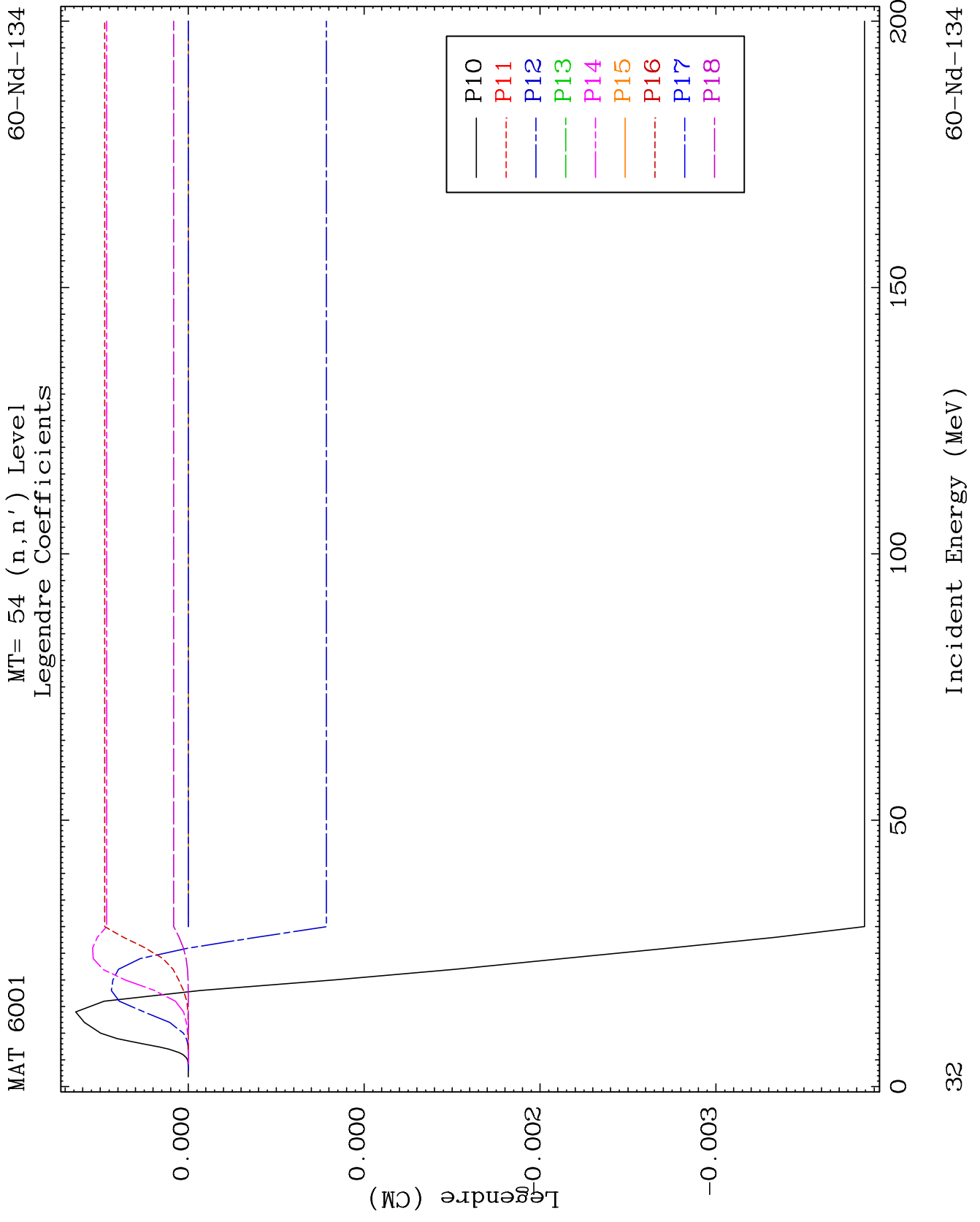


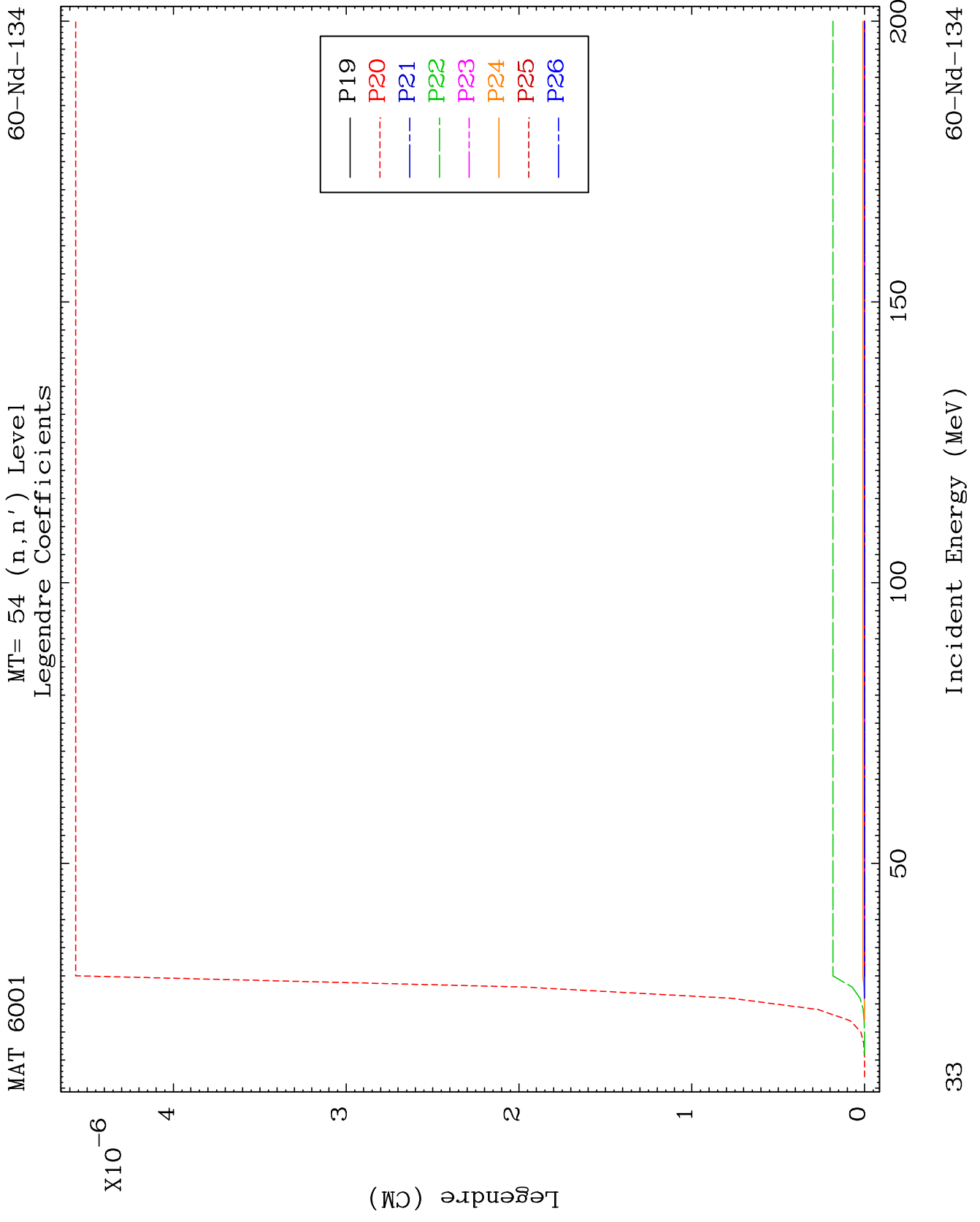


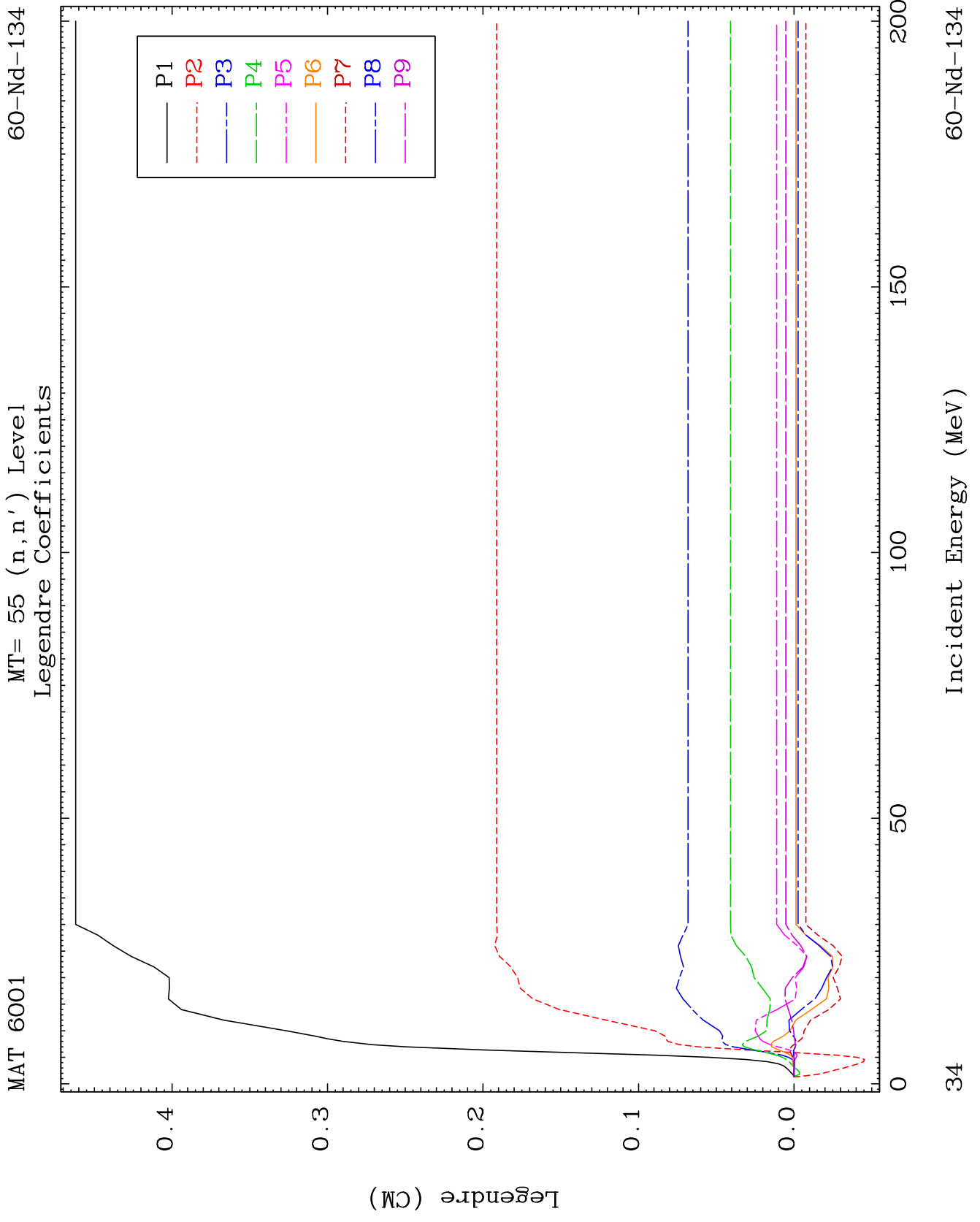


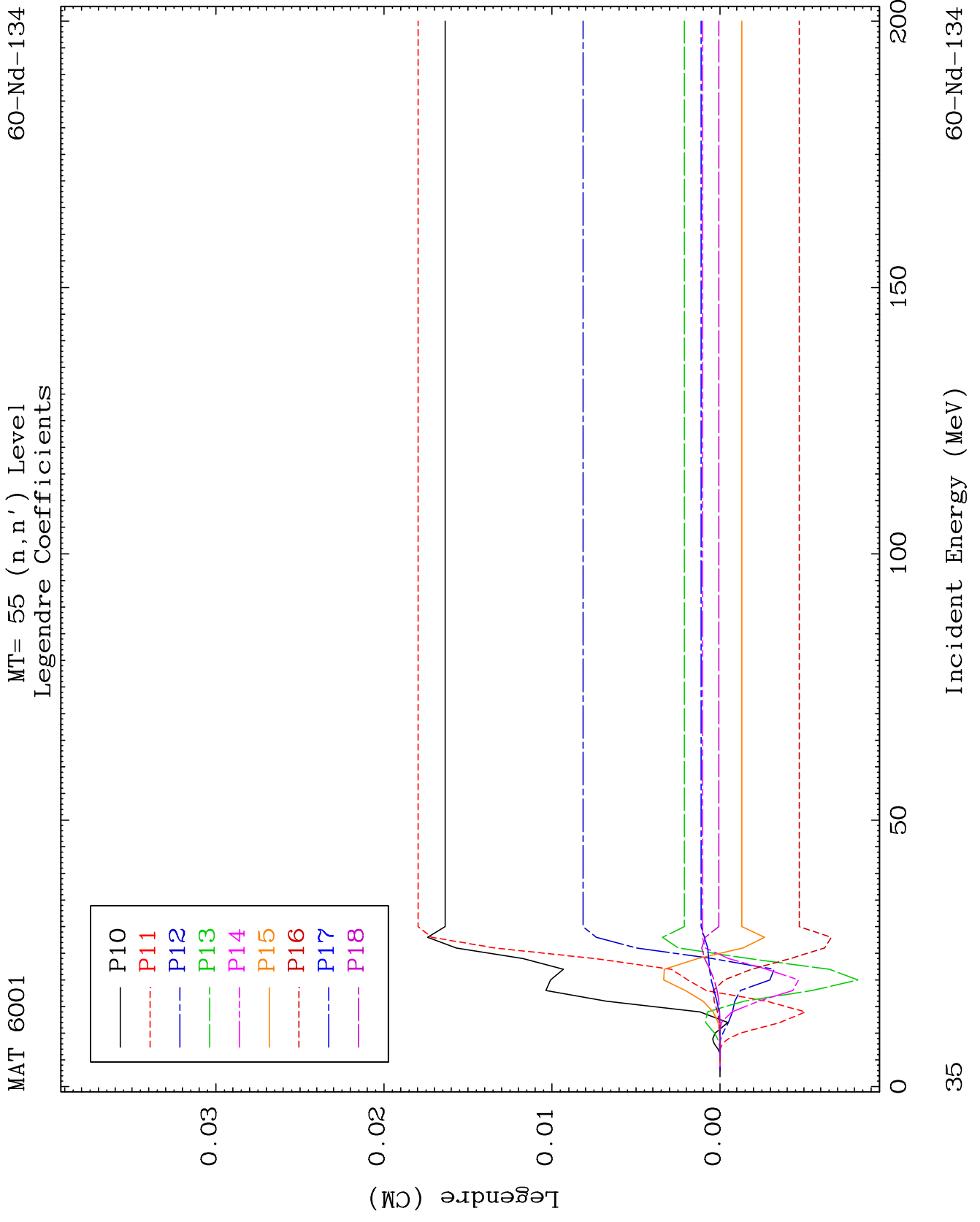








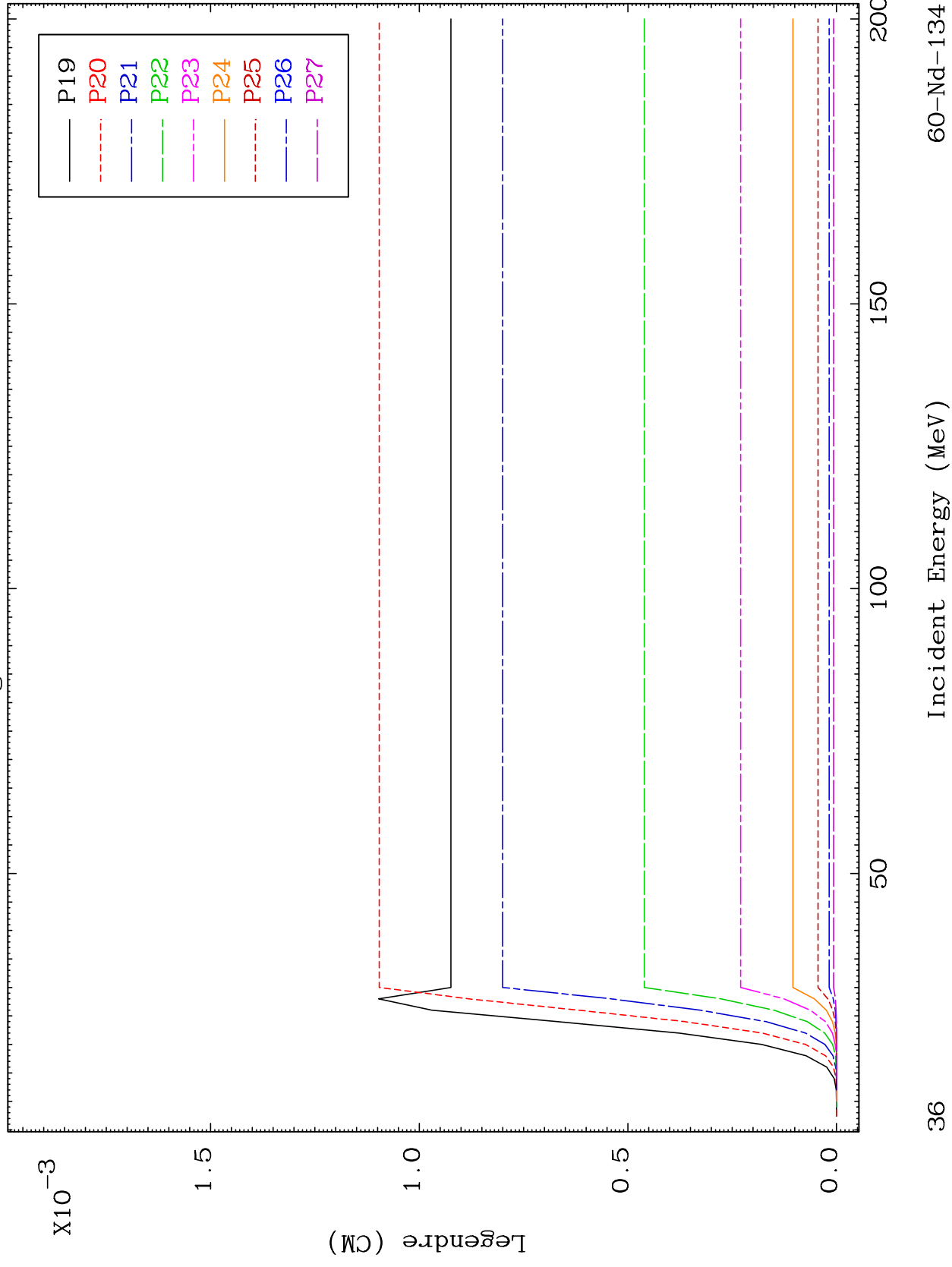




MAT 6001

MT= 55 (n,n') Level
Legendre Coefficients

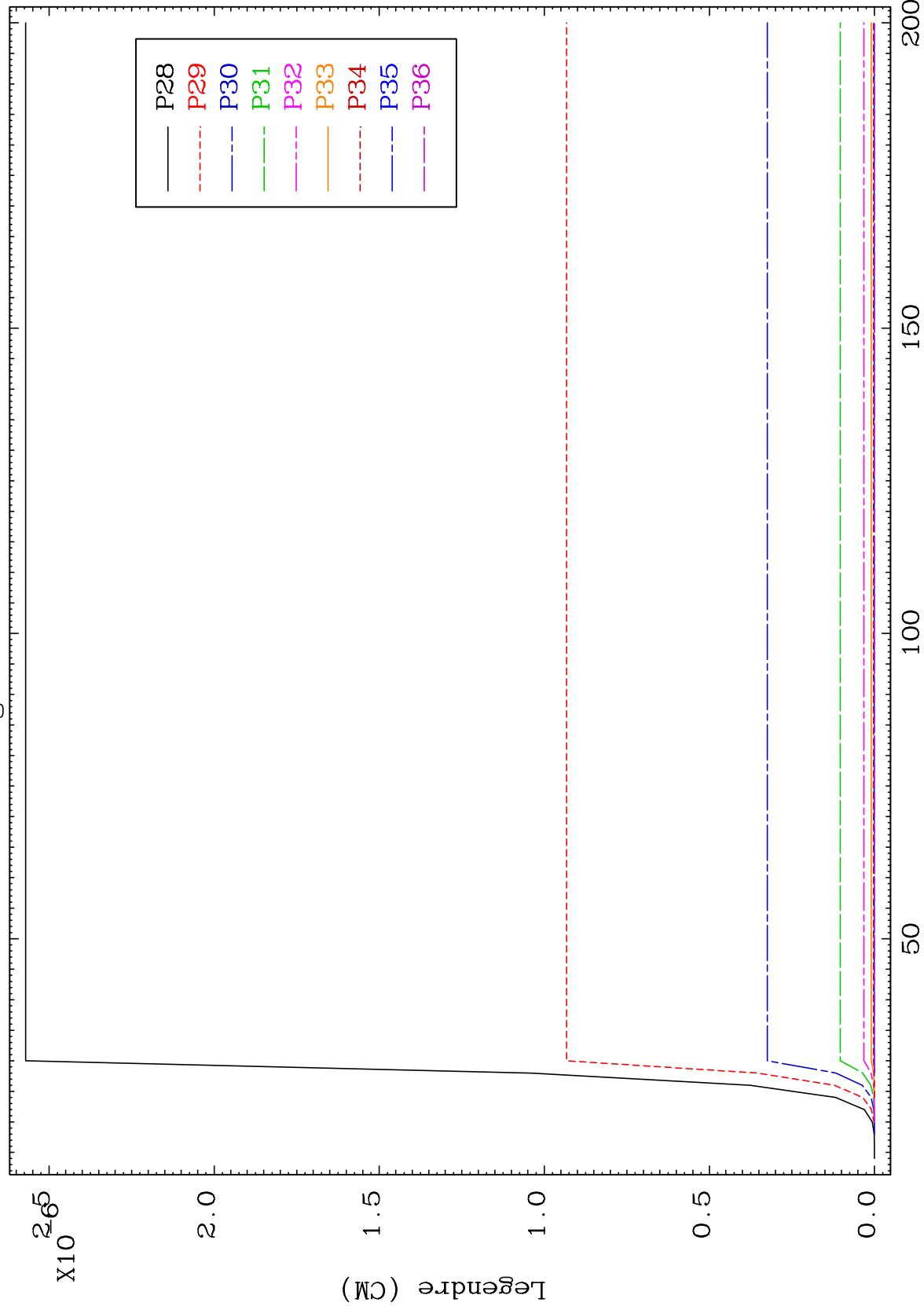
60-Nd-134



MAT 6001

MT= 55 (n,n') Level
Legendre Coefficients

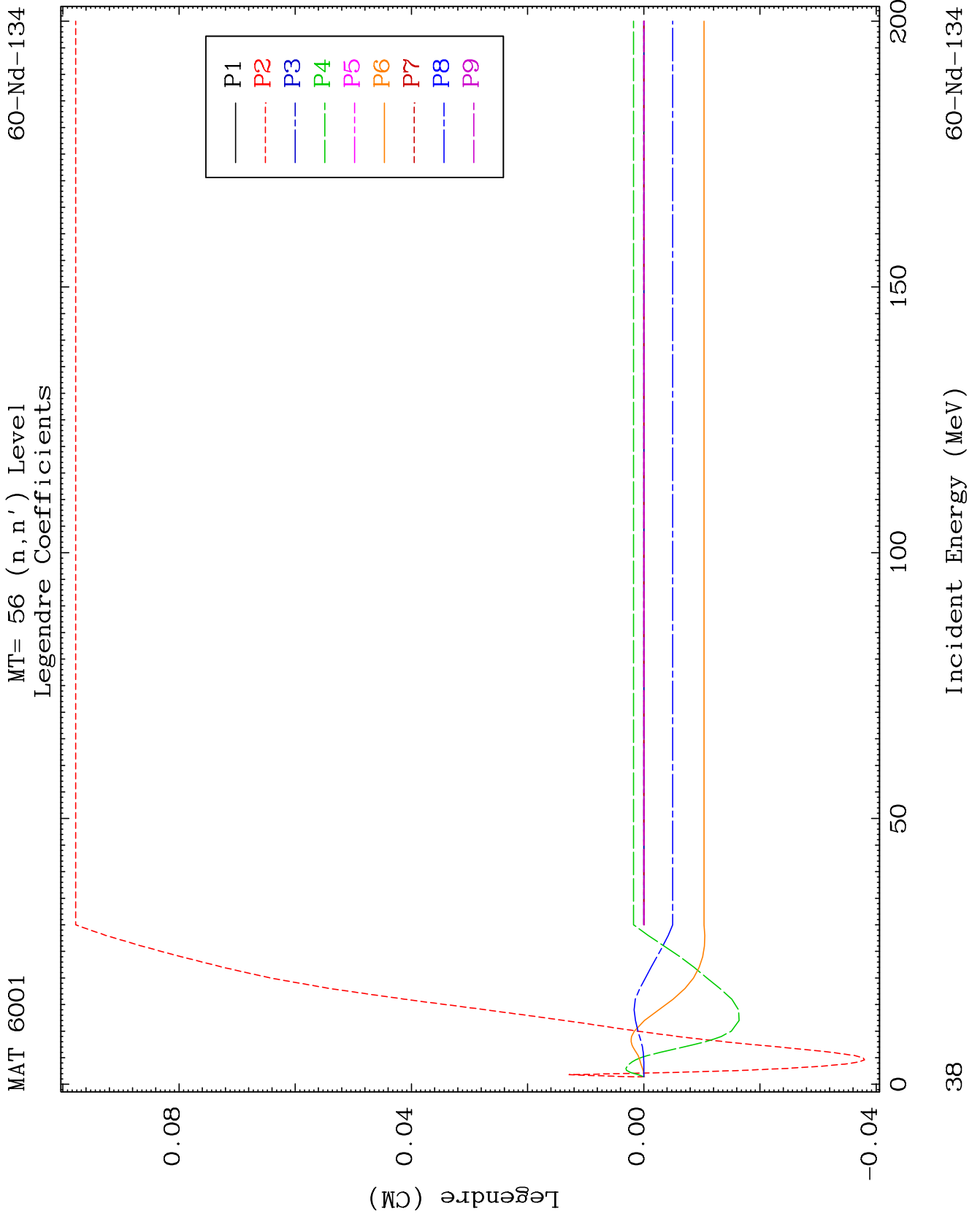
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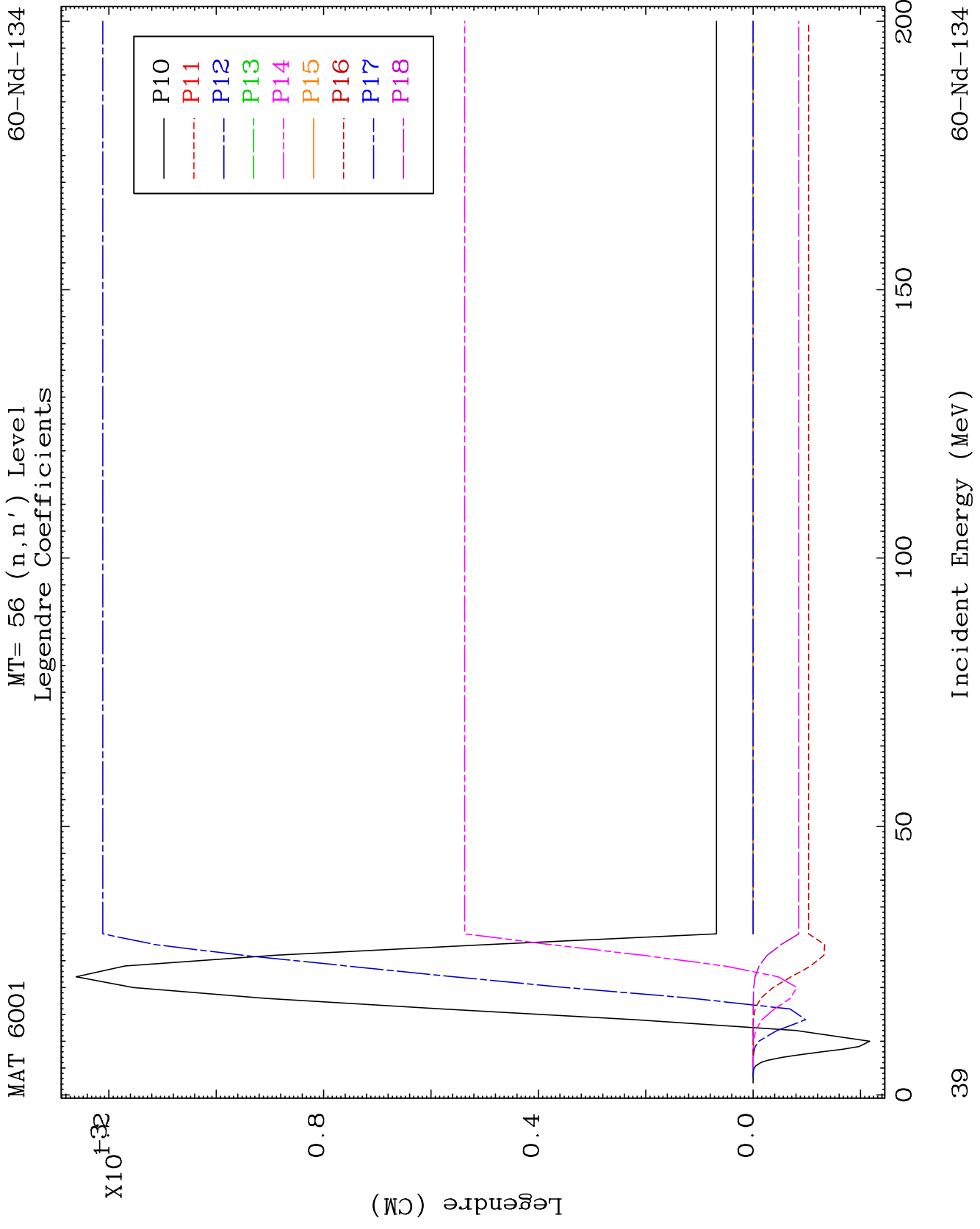


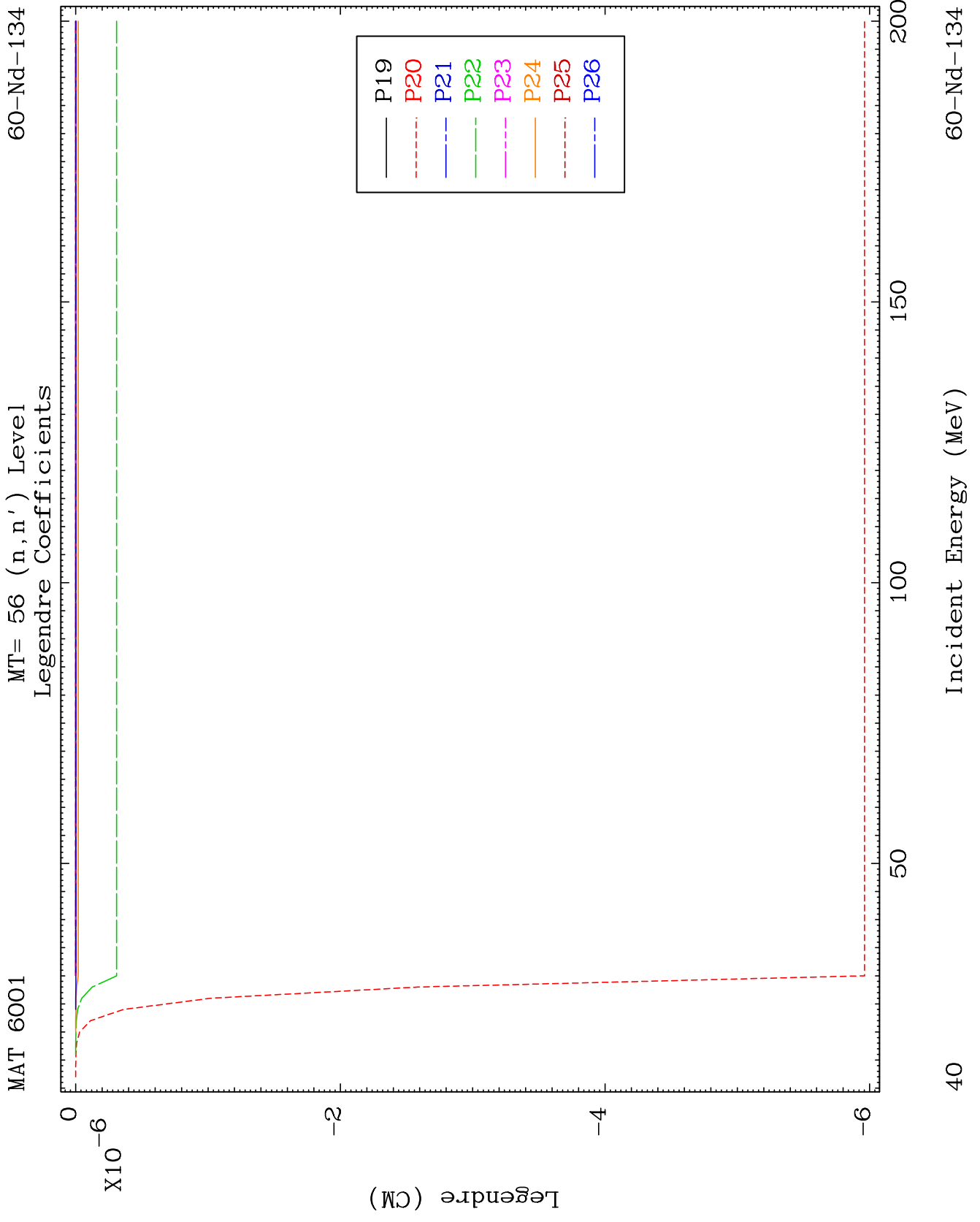
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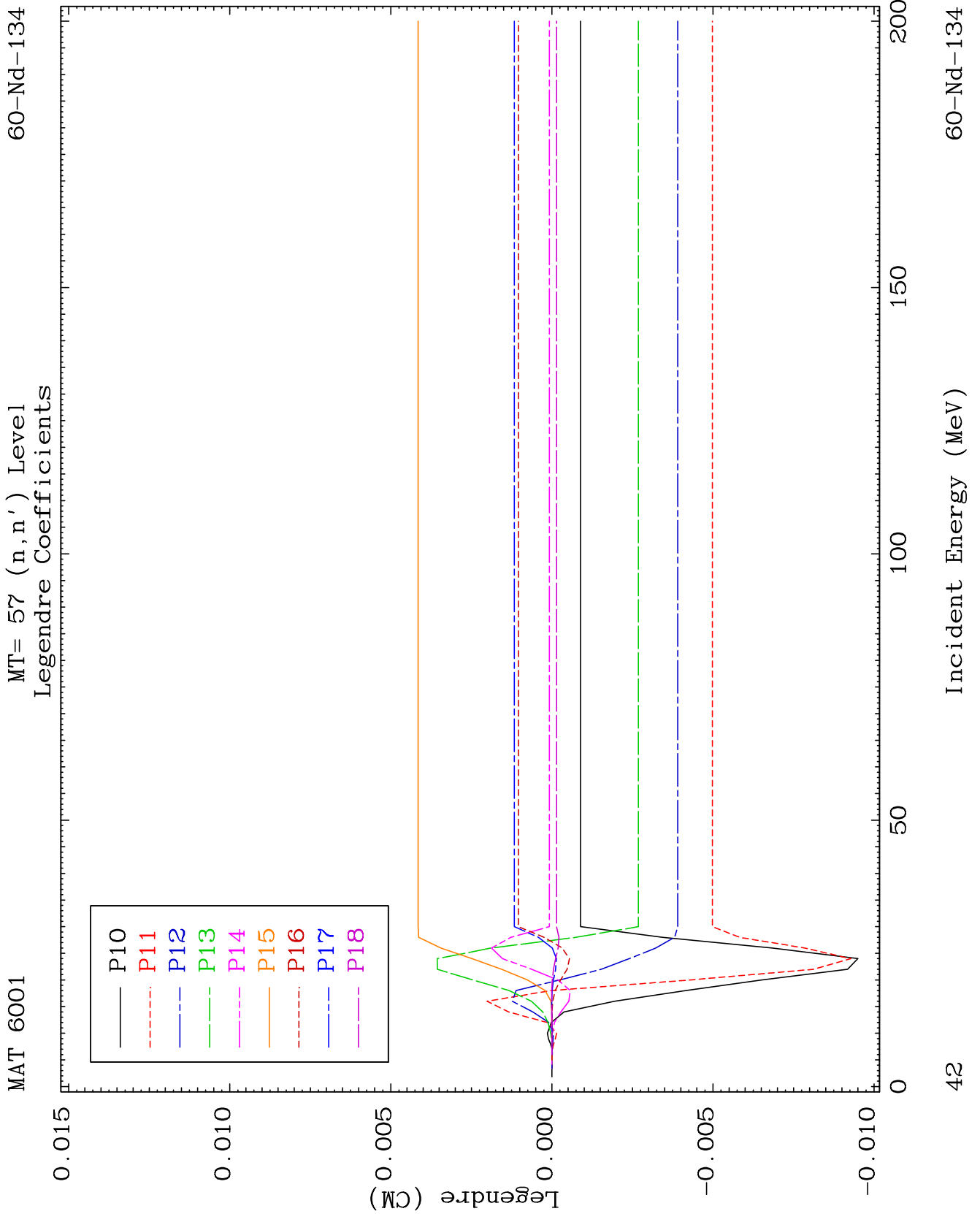
Incident Energy (MeV)

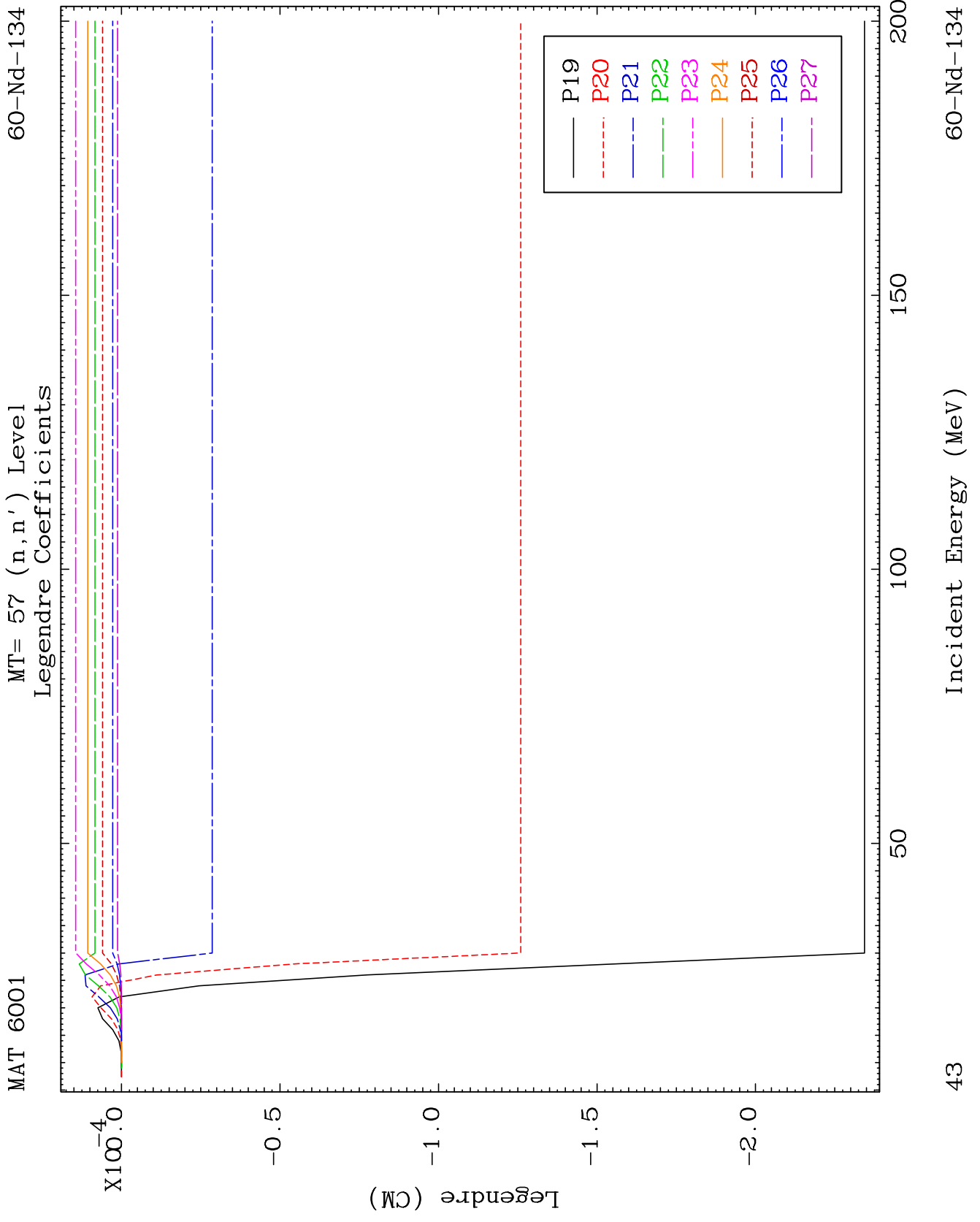
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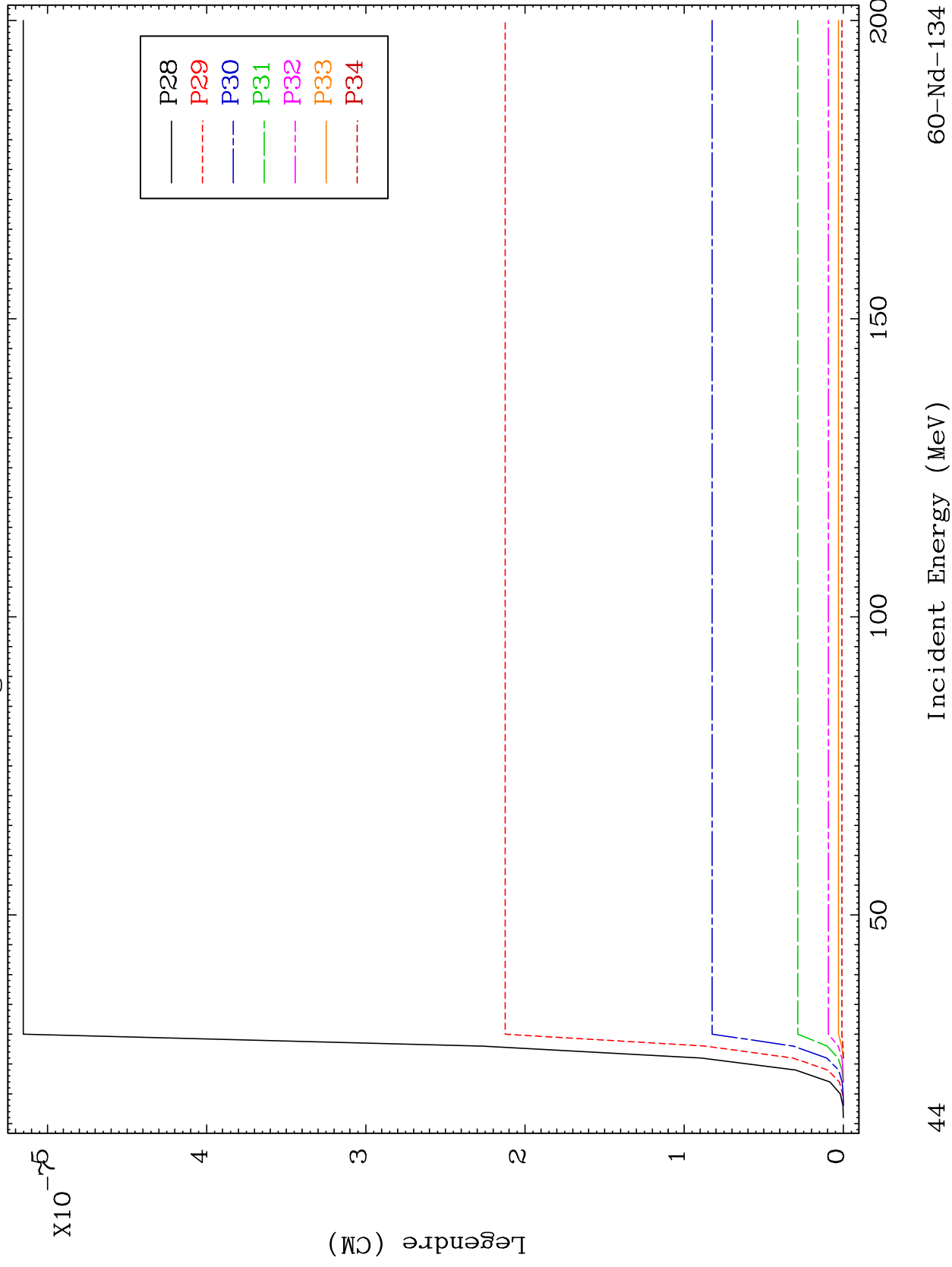




MAT 6001

MT= 57 (n,n') Level
Legendre Coefficients

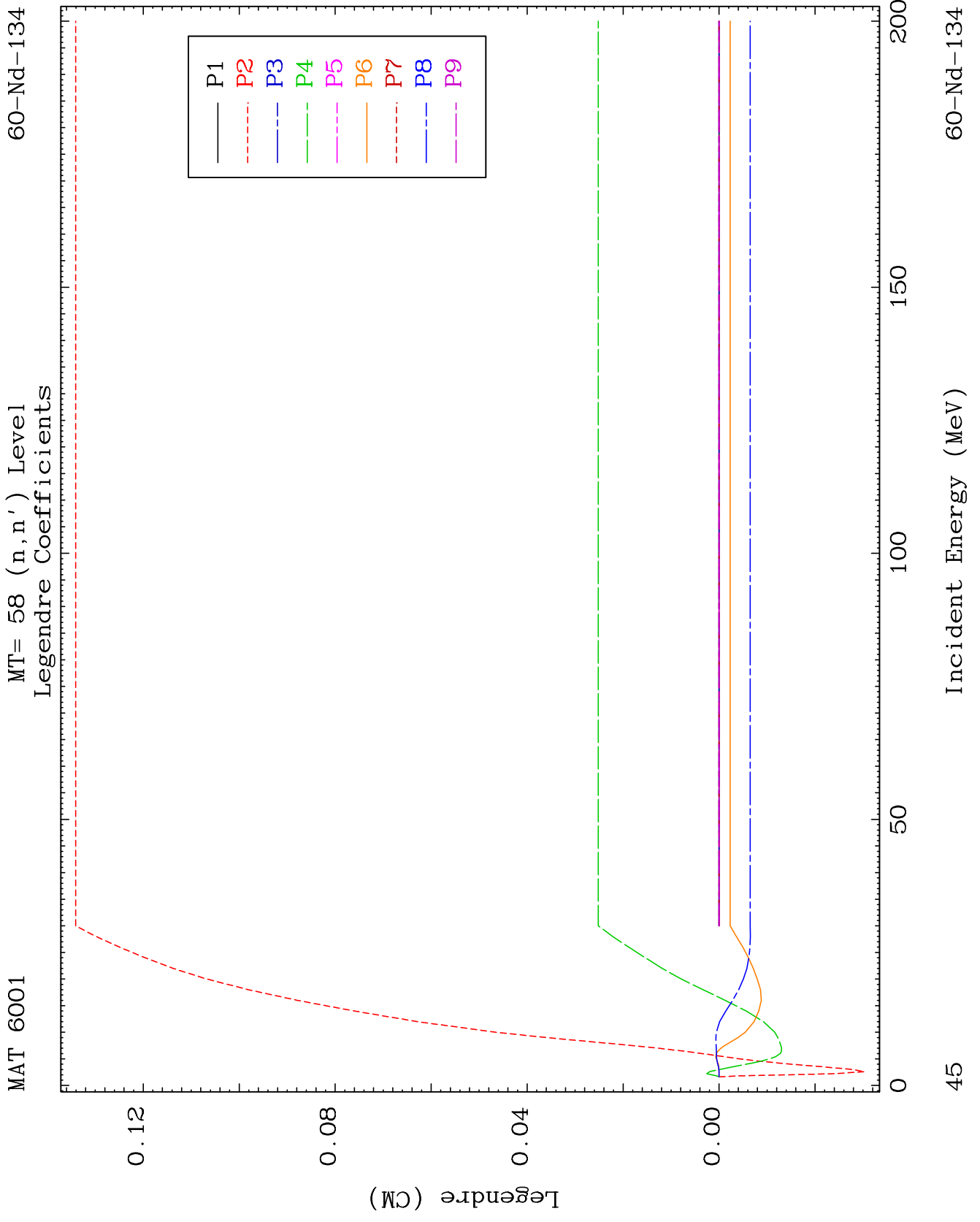
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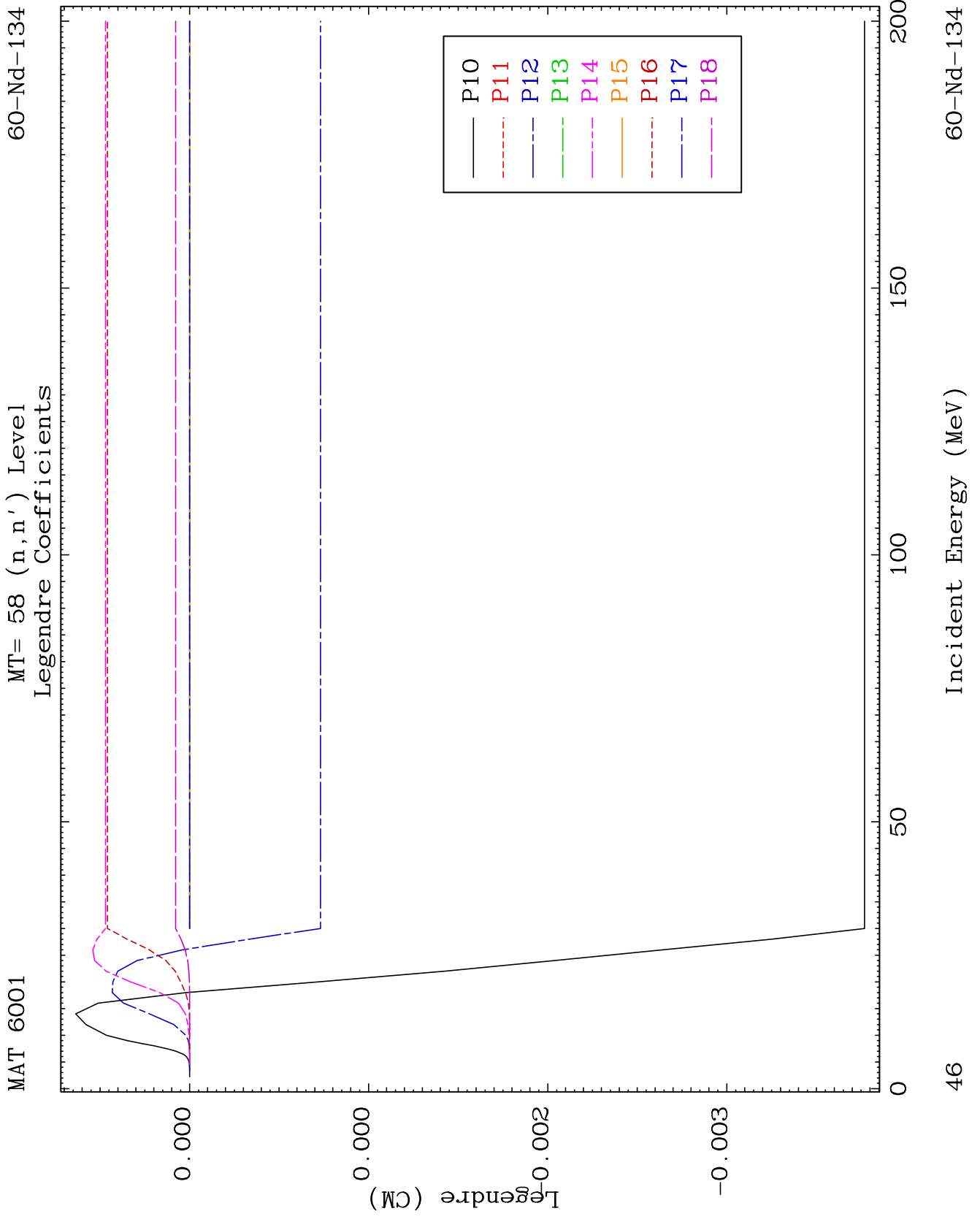


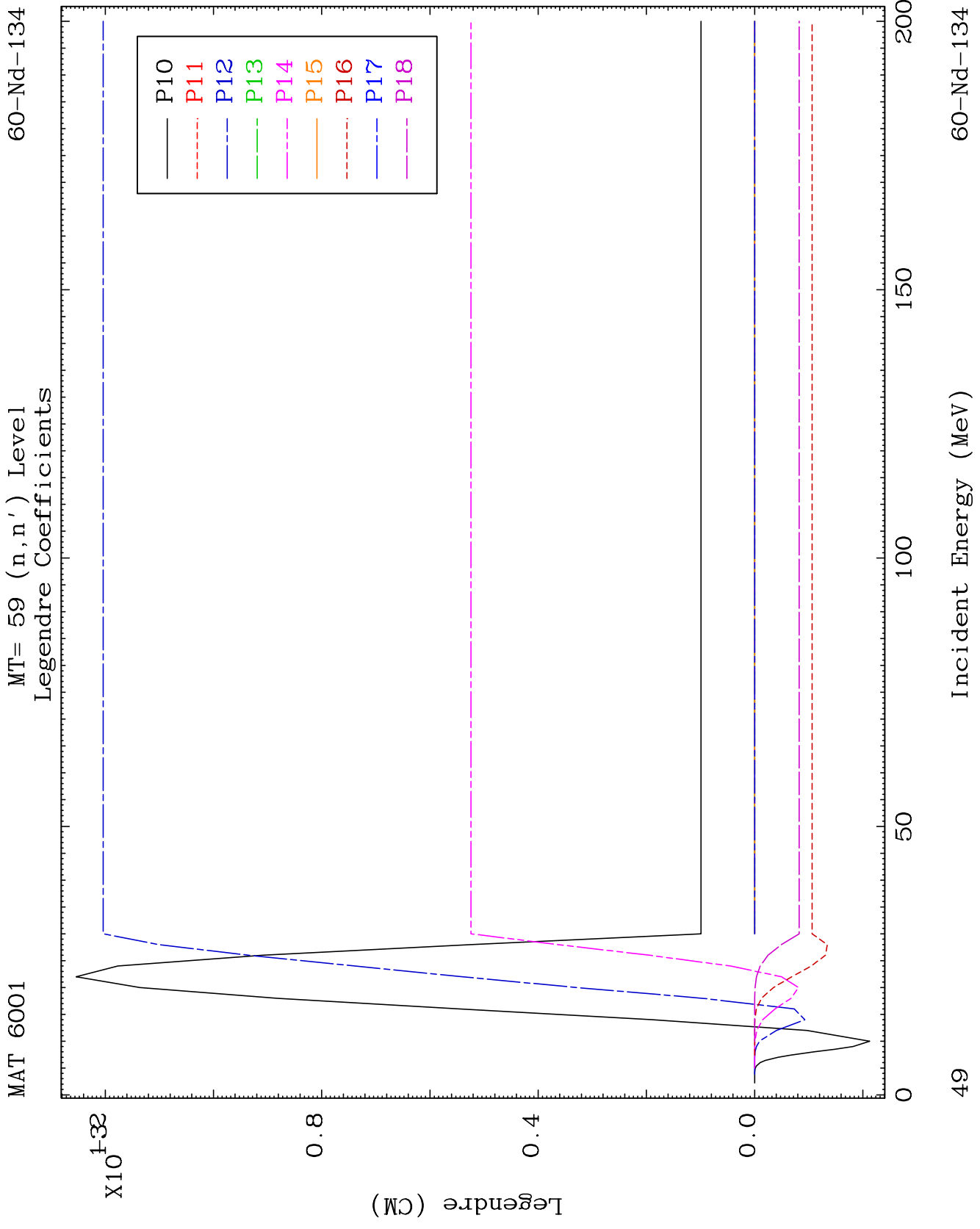
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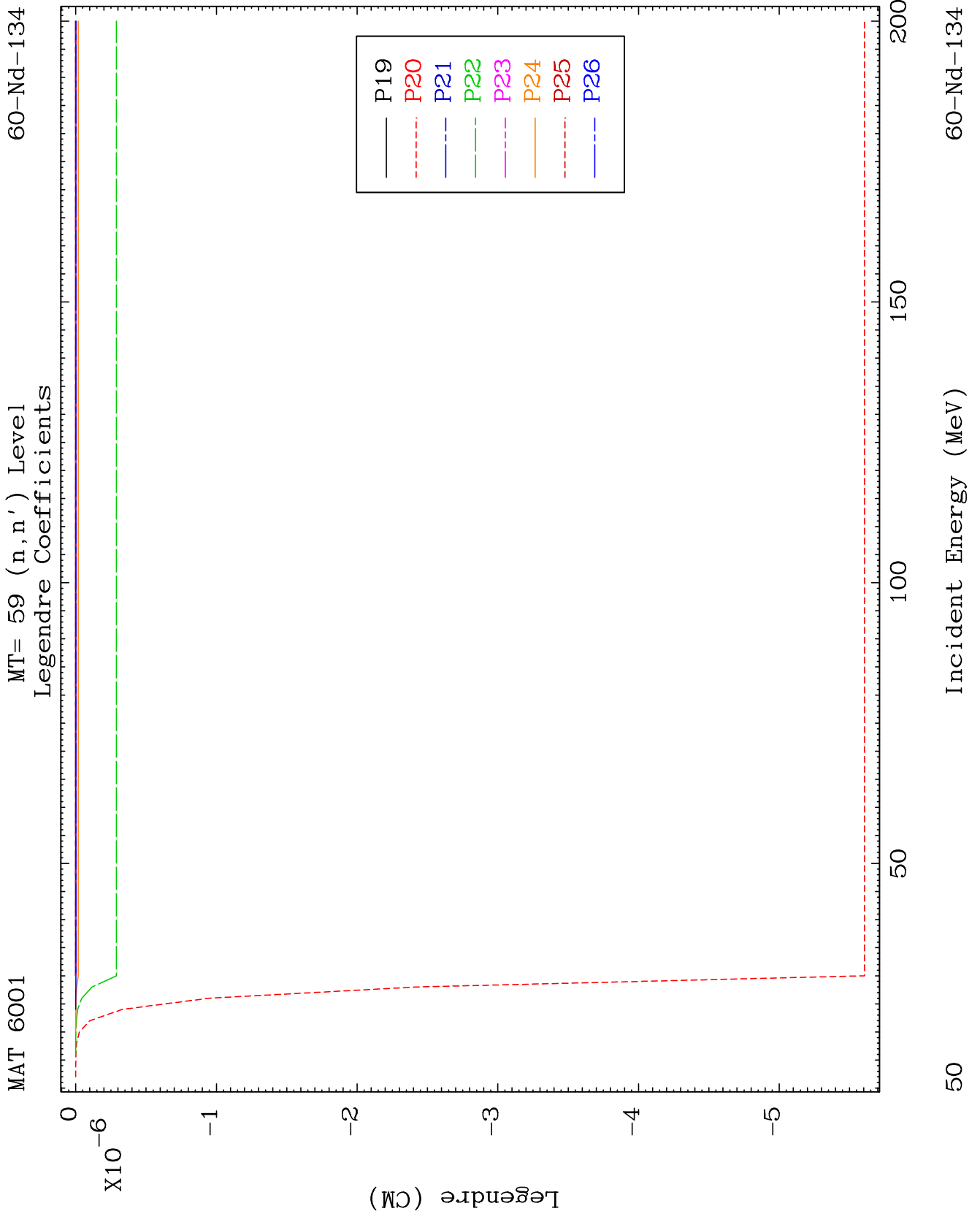
Incident Energy (MeV)

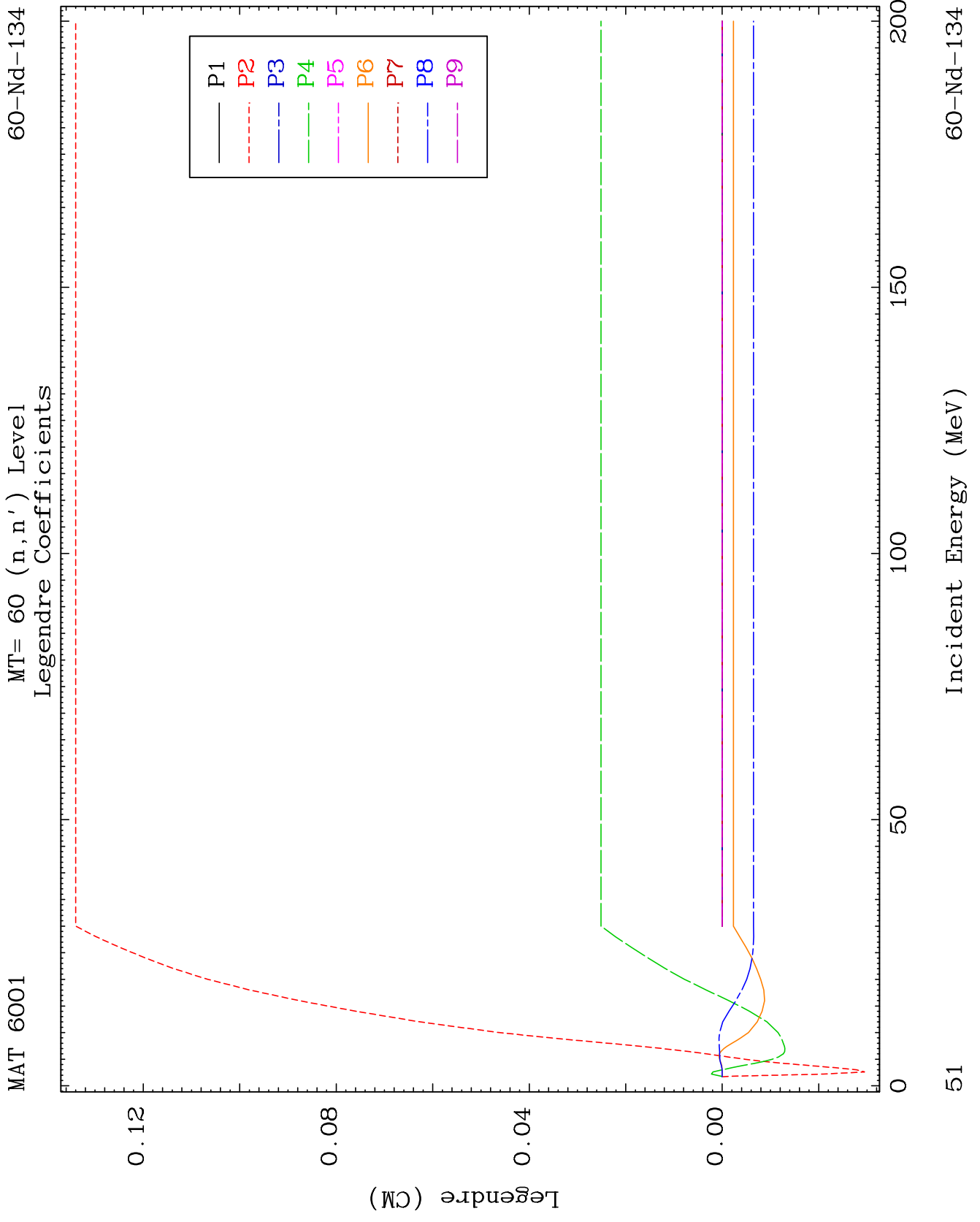
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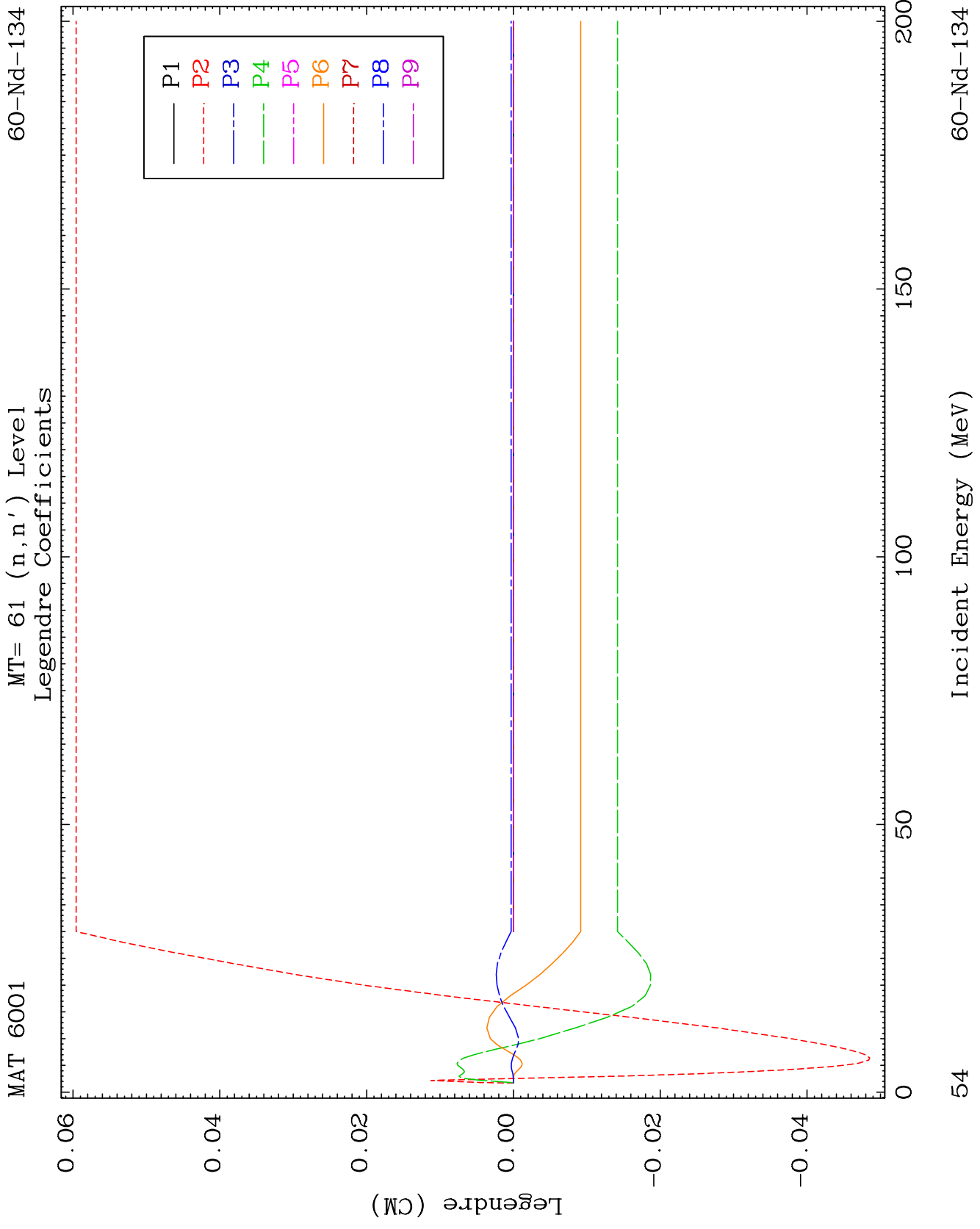








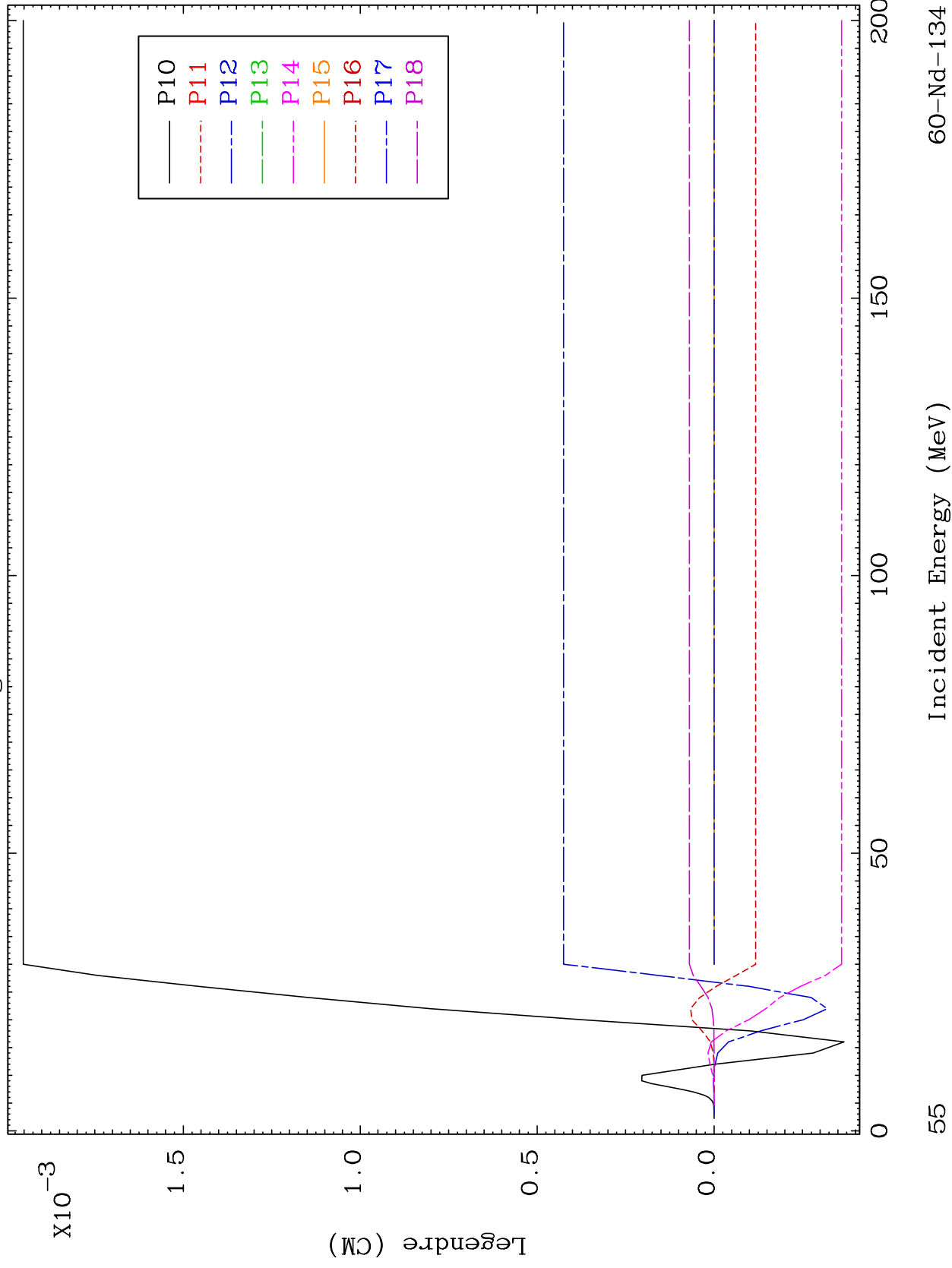




MAT 6001

MT= 61 (n,n') Level
Legendre Coefficients

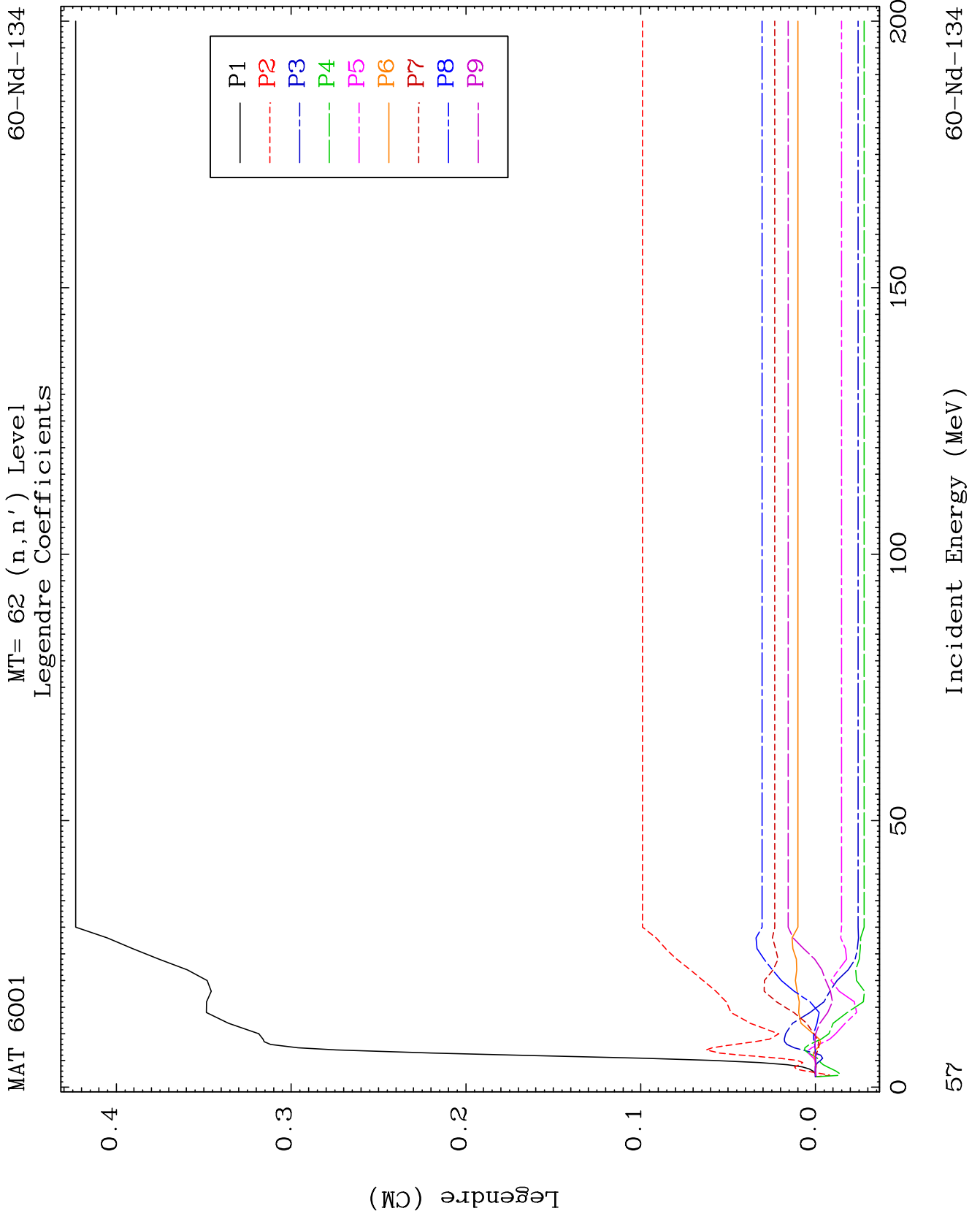
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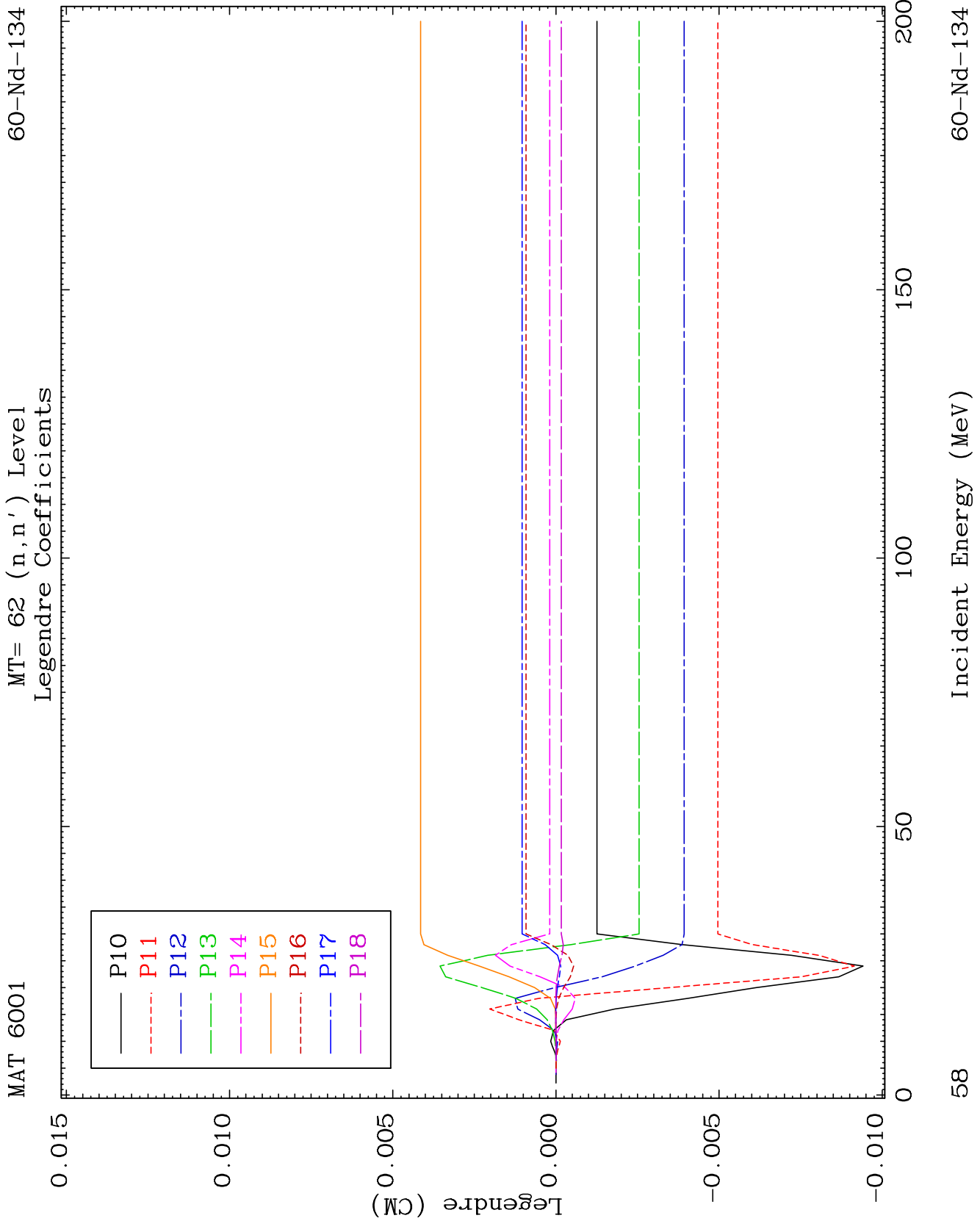


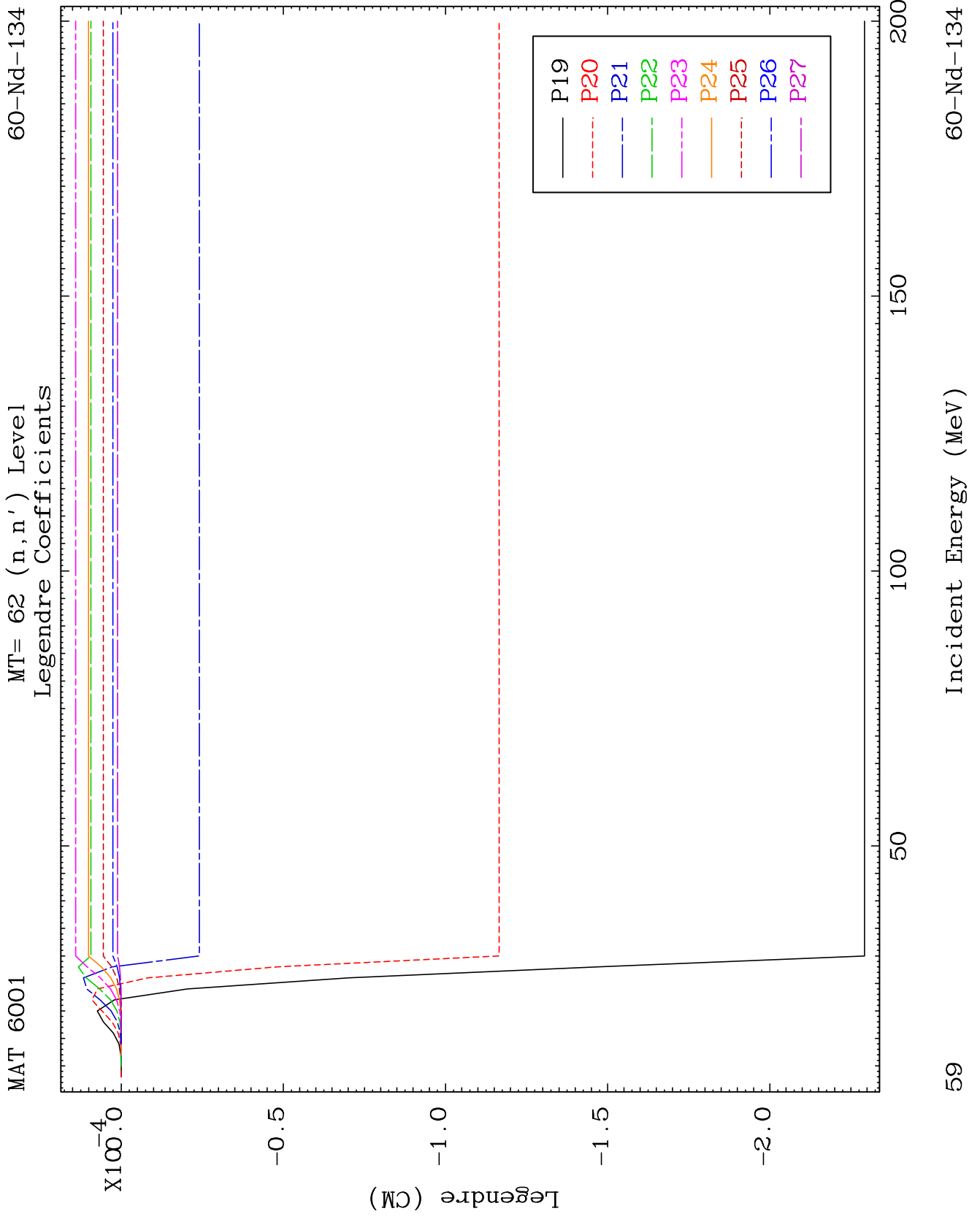
60-Nd-134

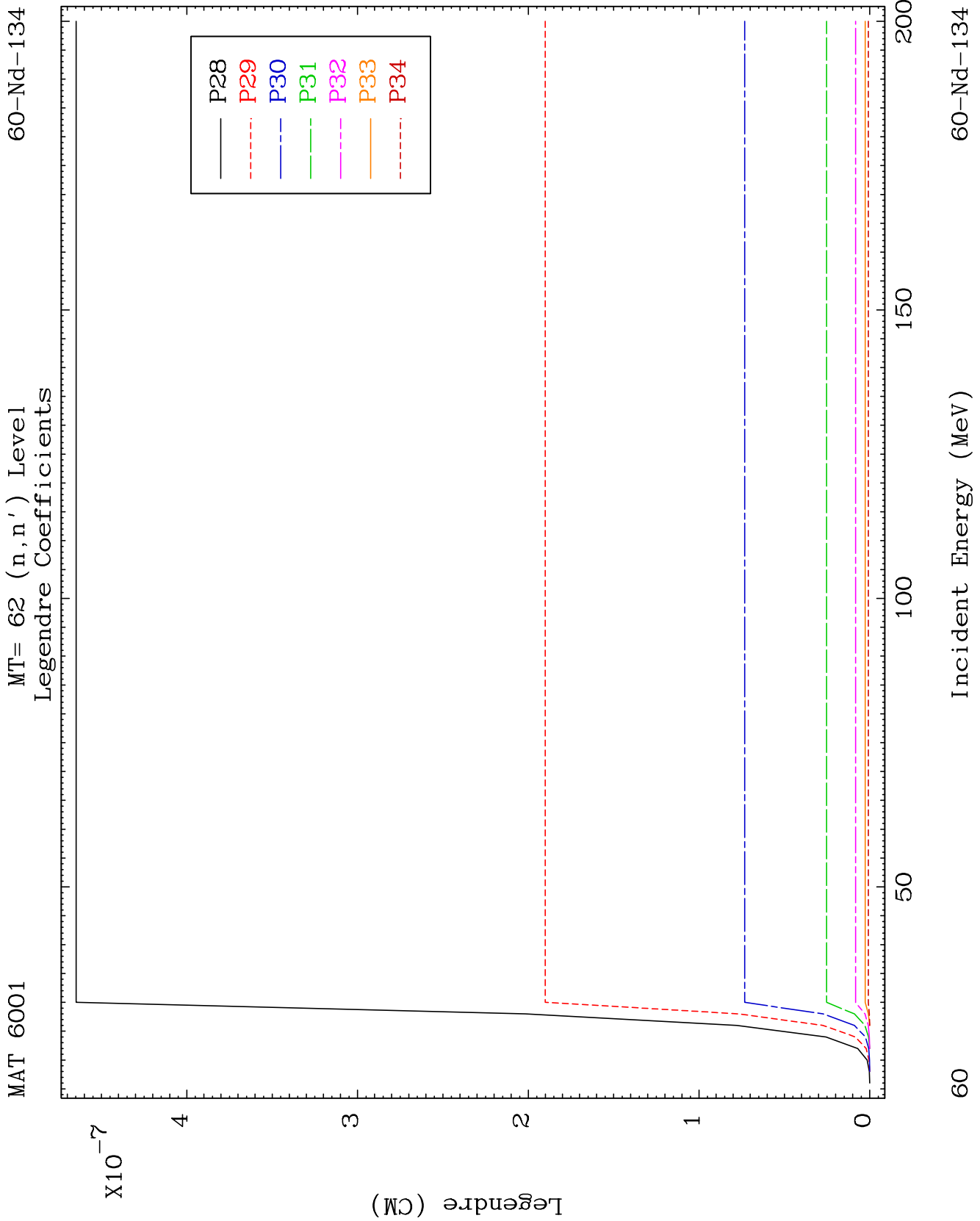
Incident Energy (MeV)

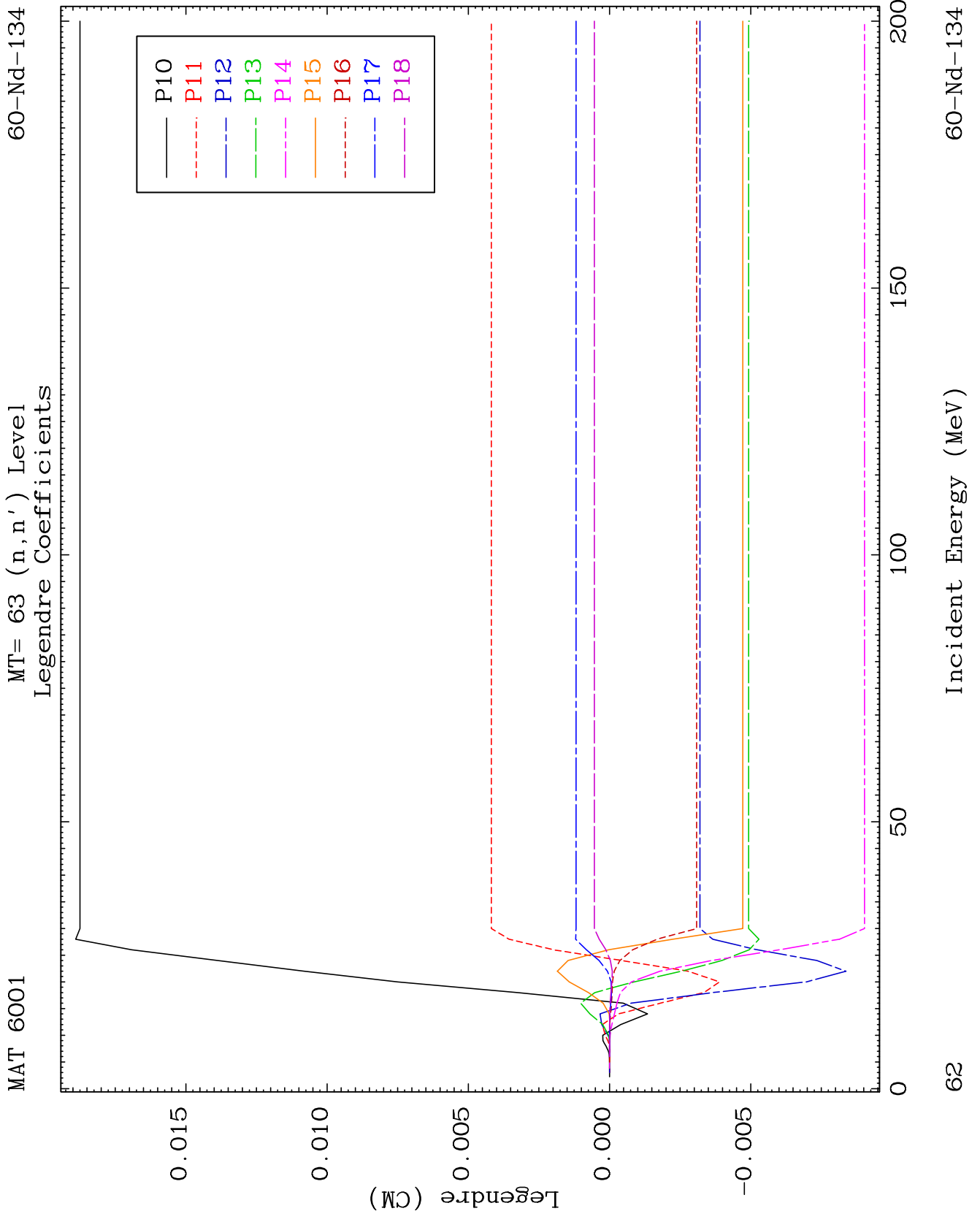
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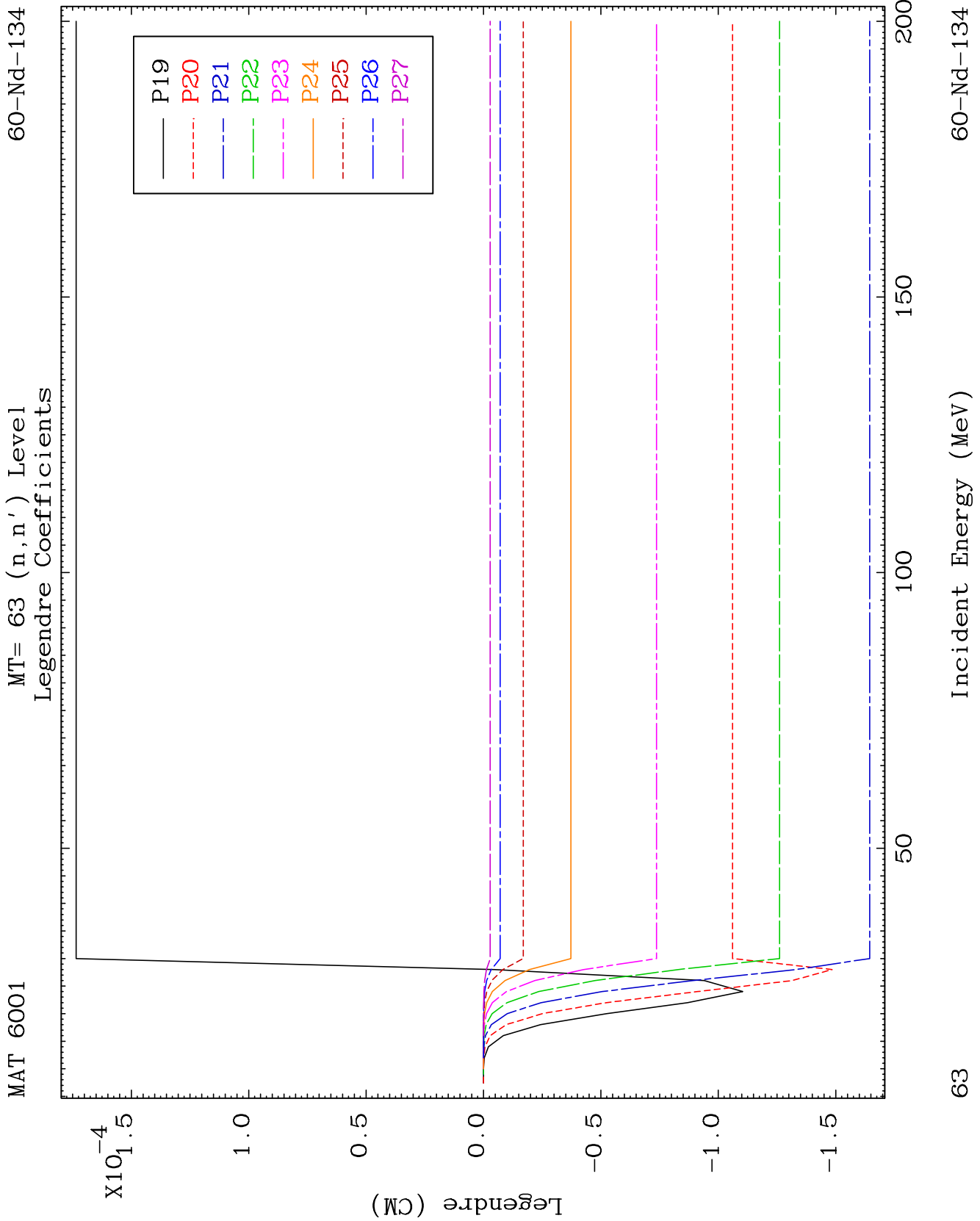


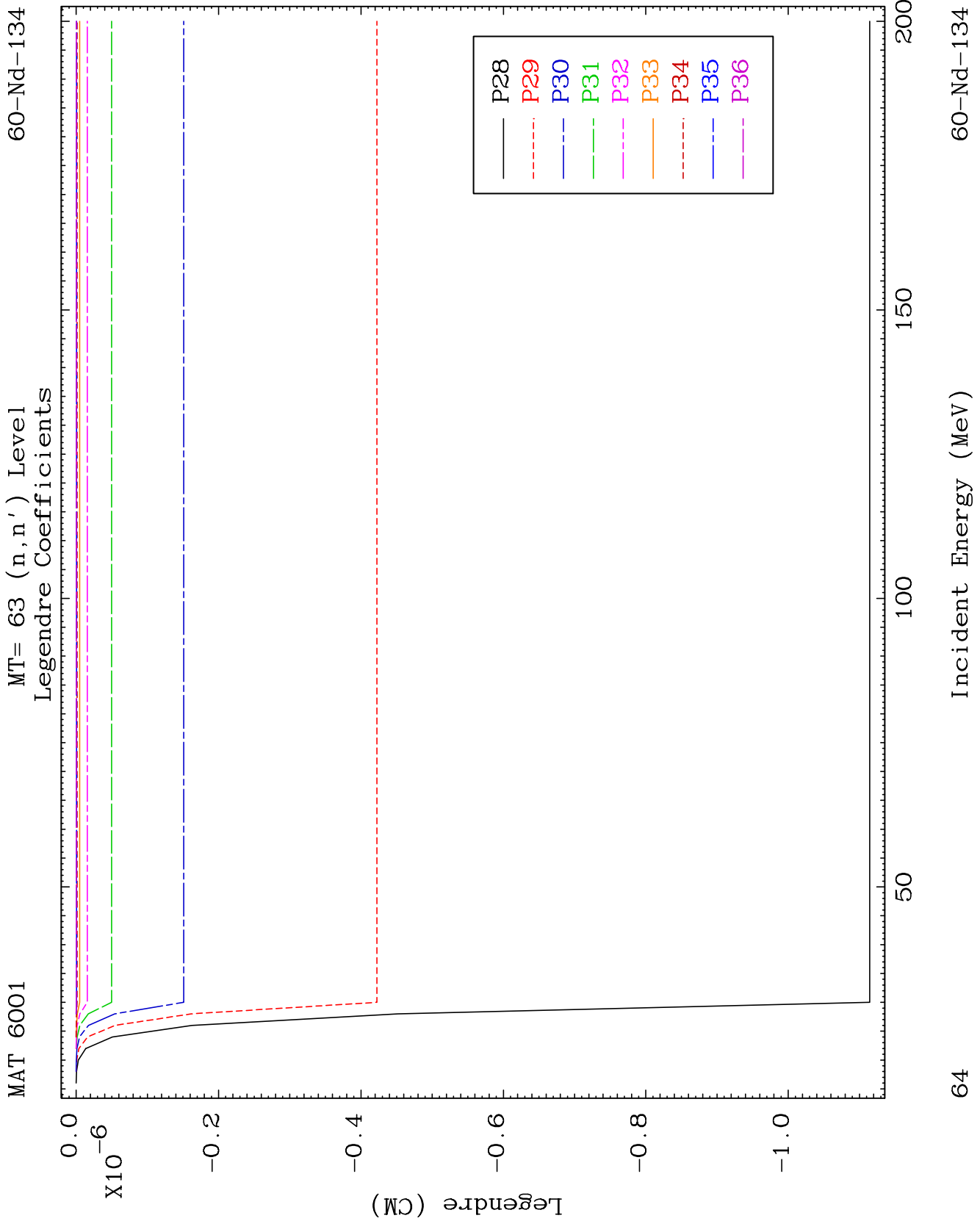


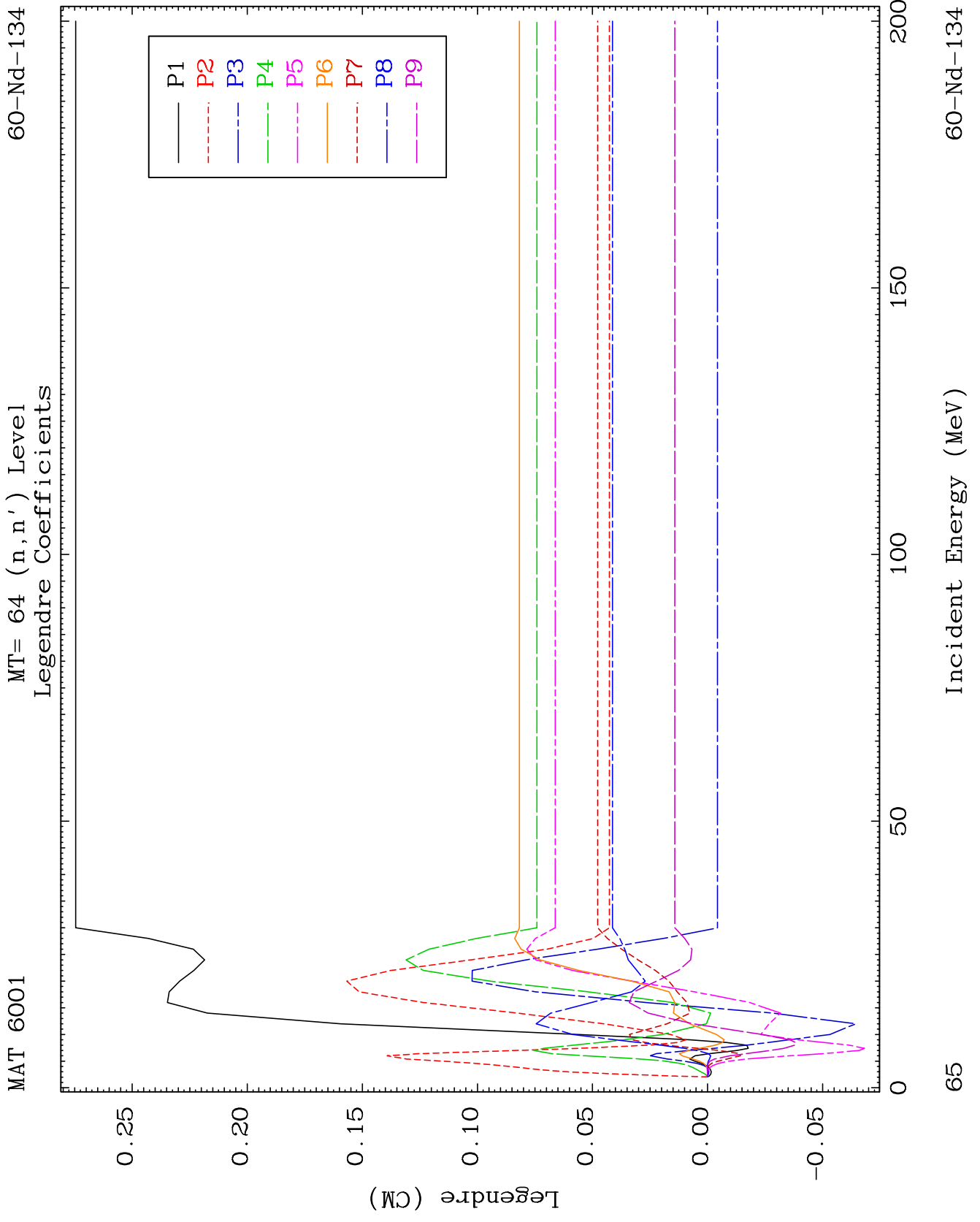


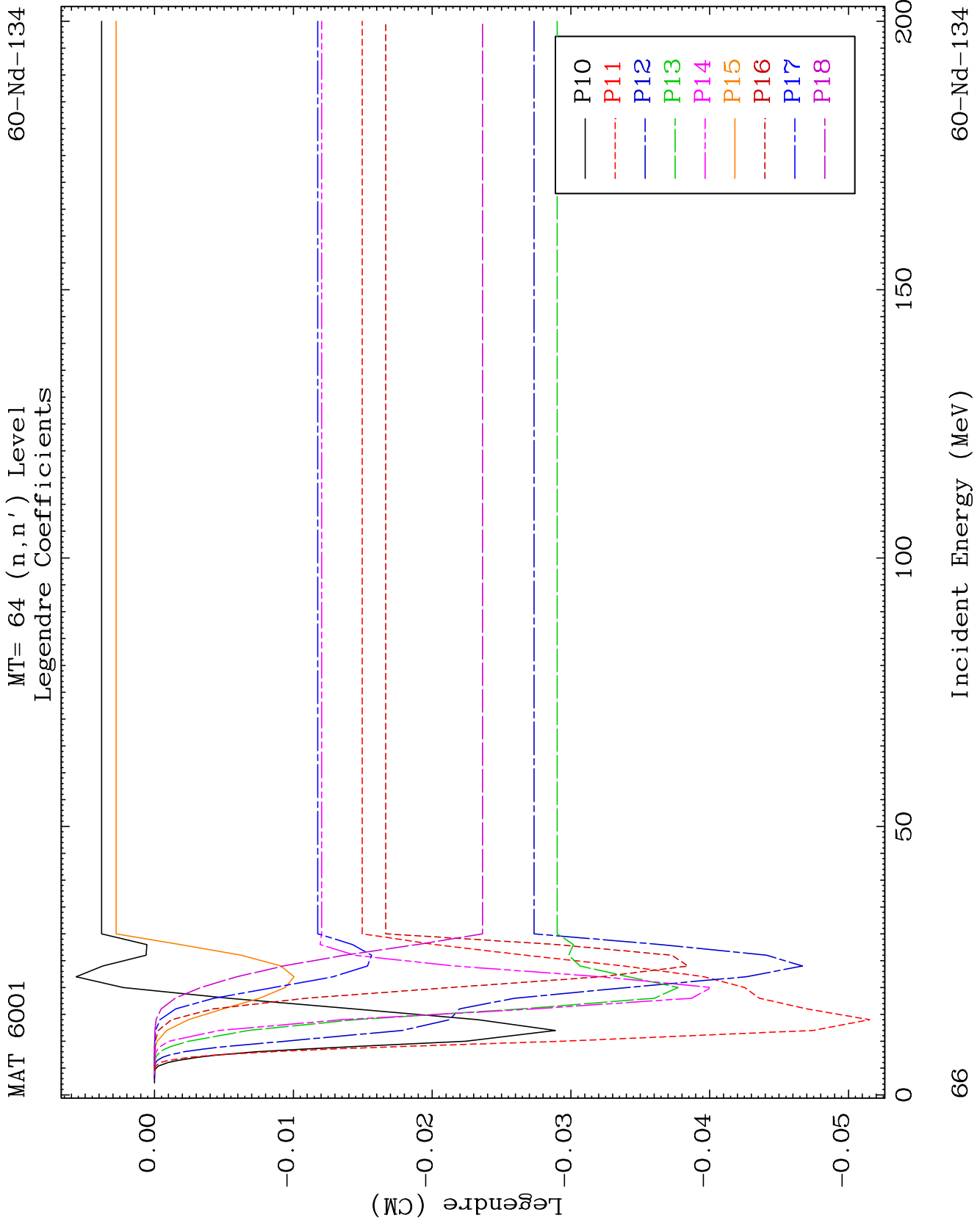


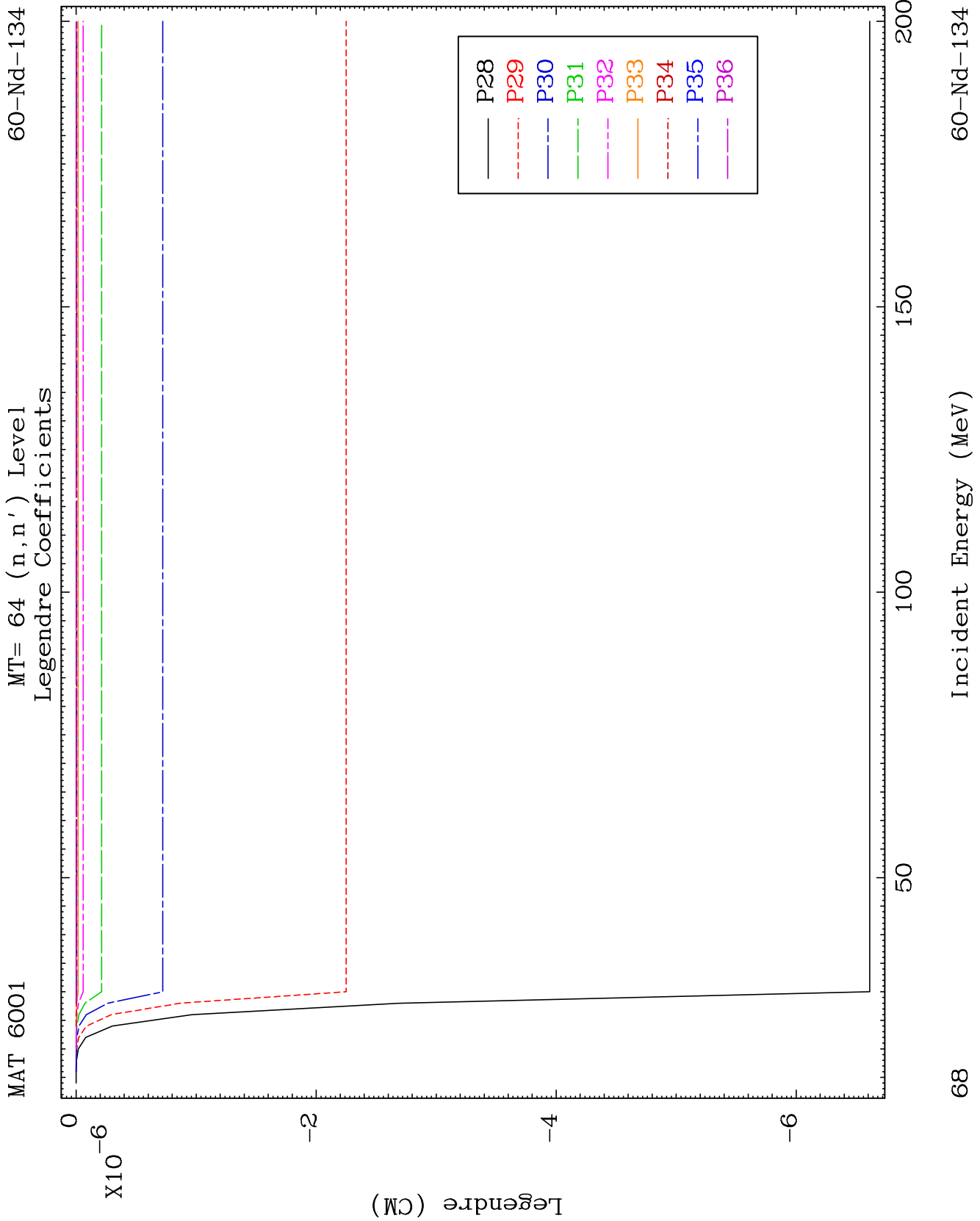


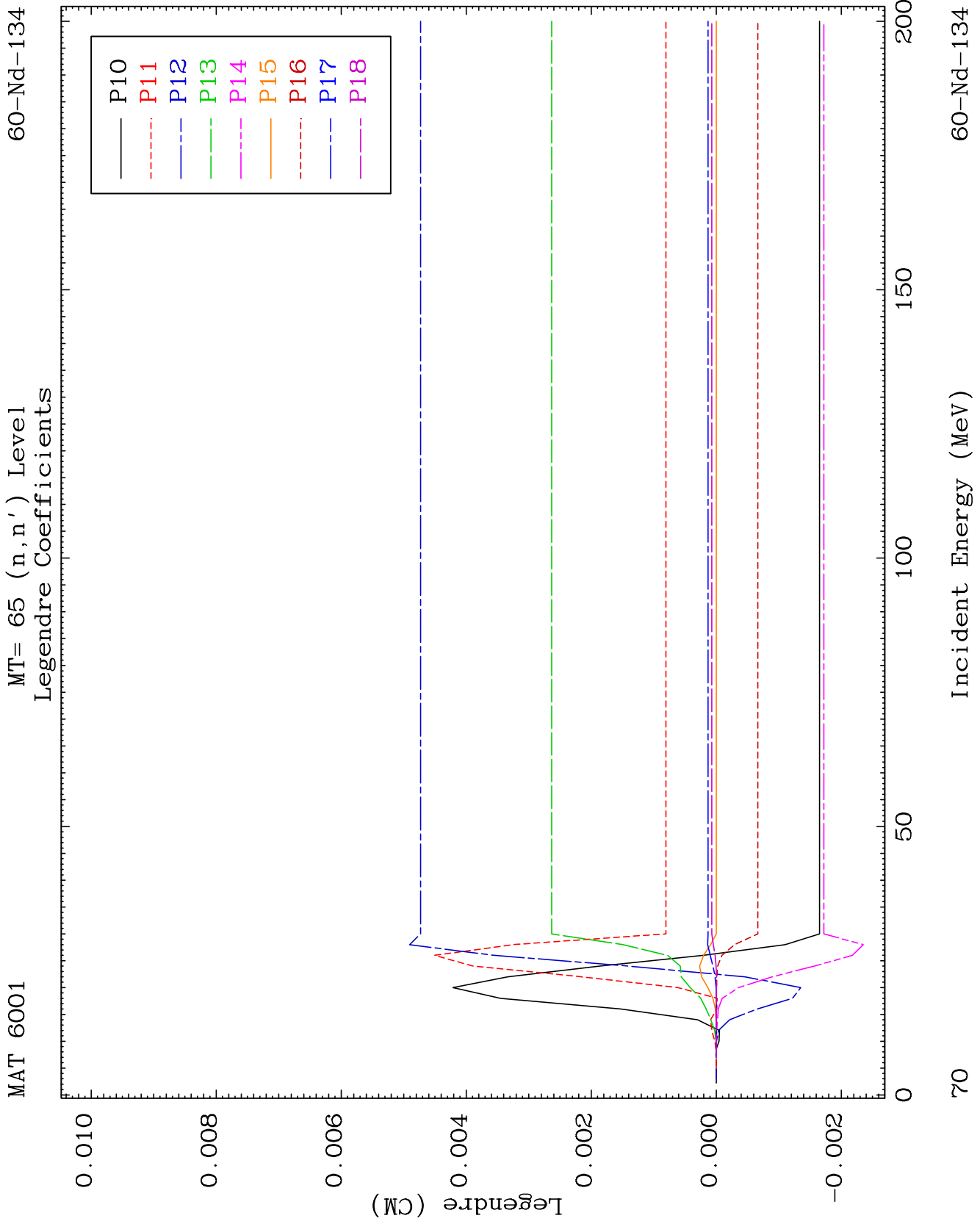








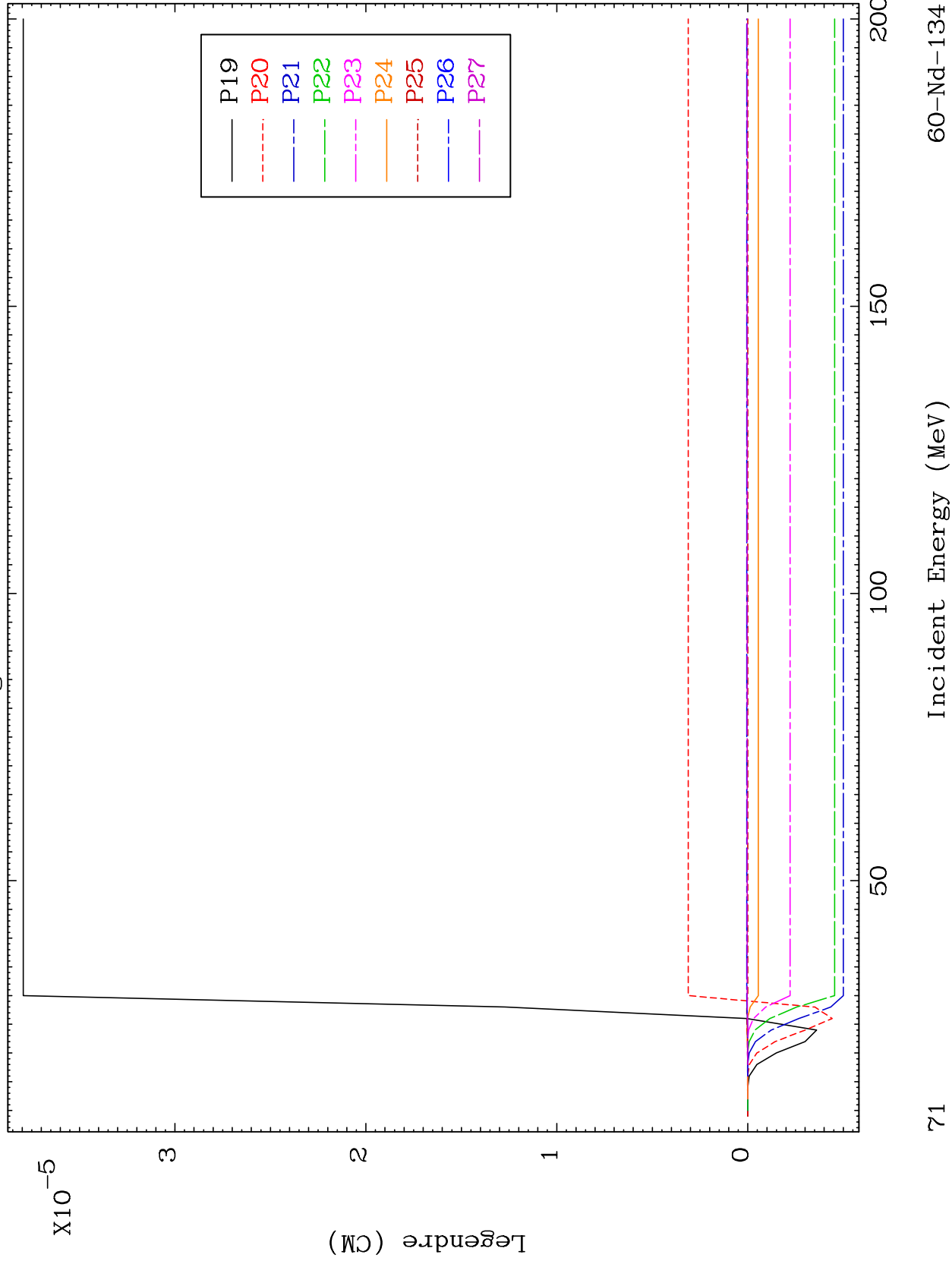


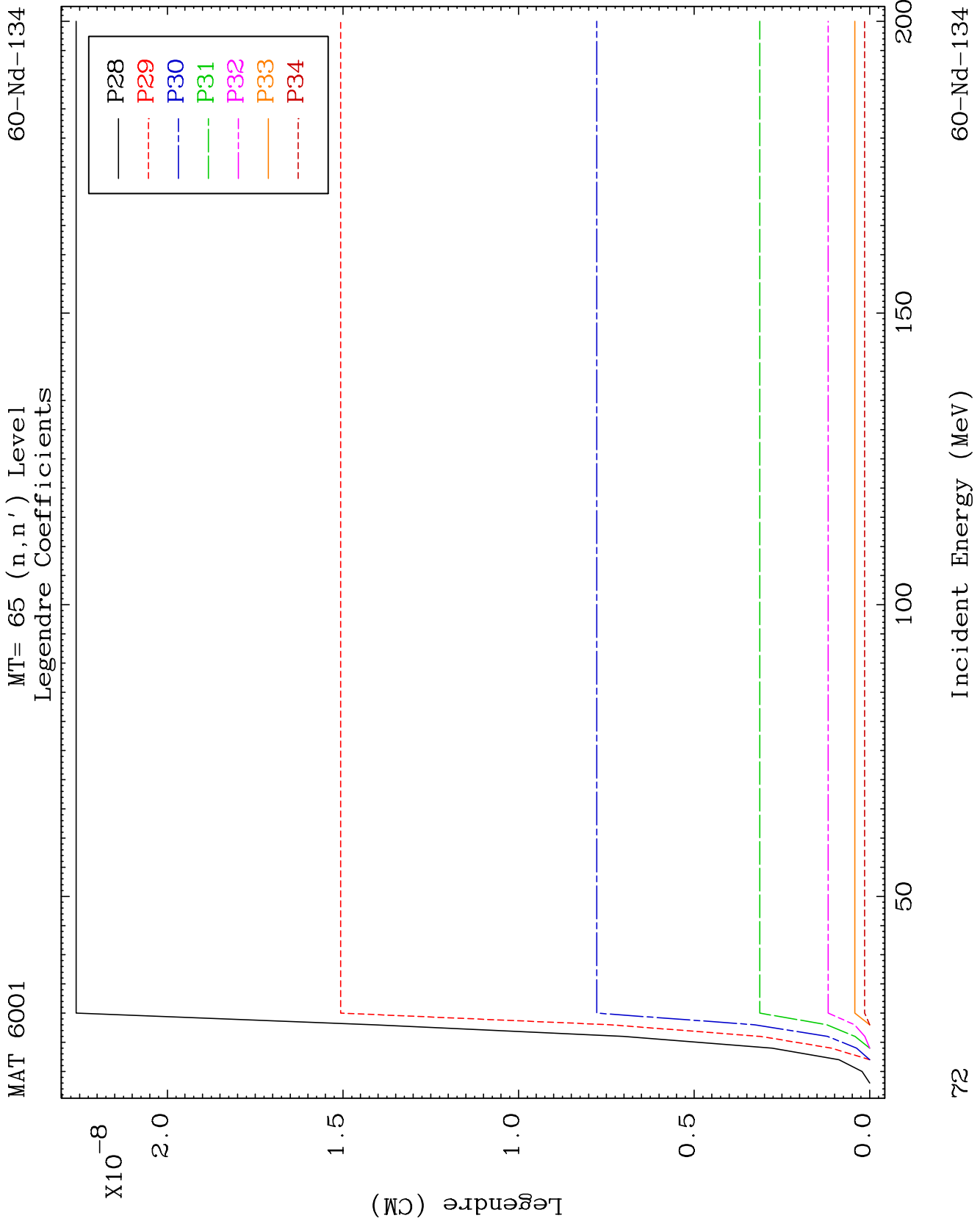


MAT 6001

MT= 65 (n,n') Level
Legendre Coefficients

60-Nd-134

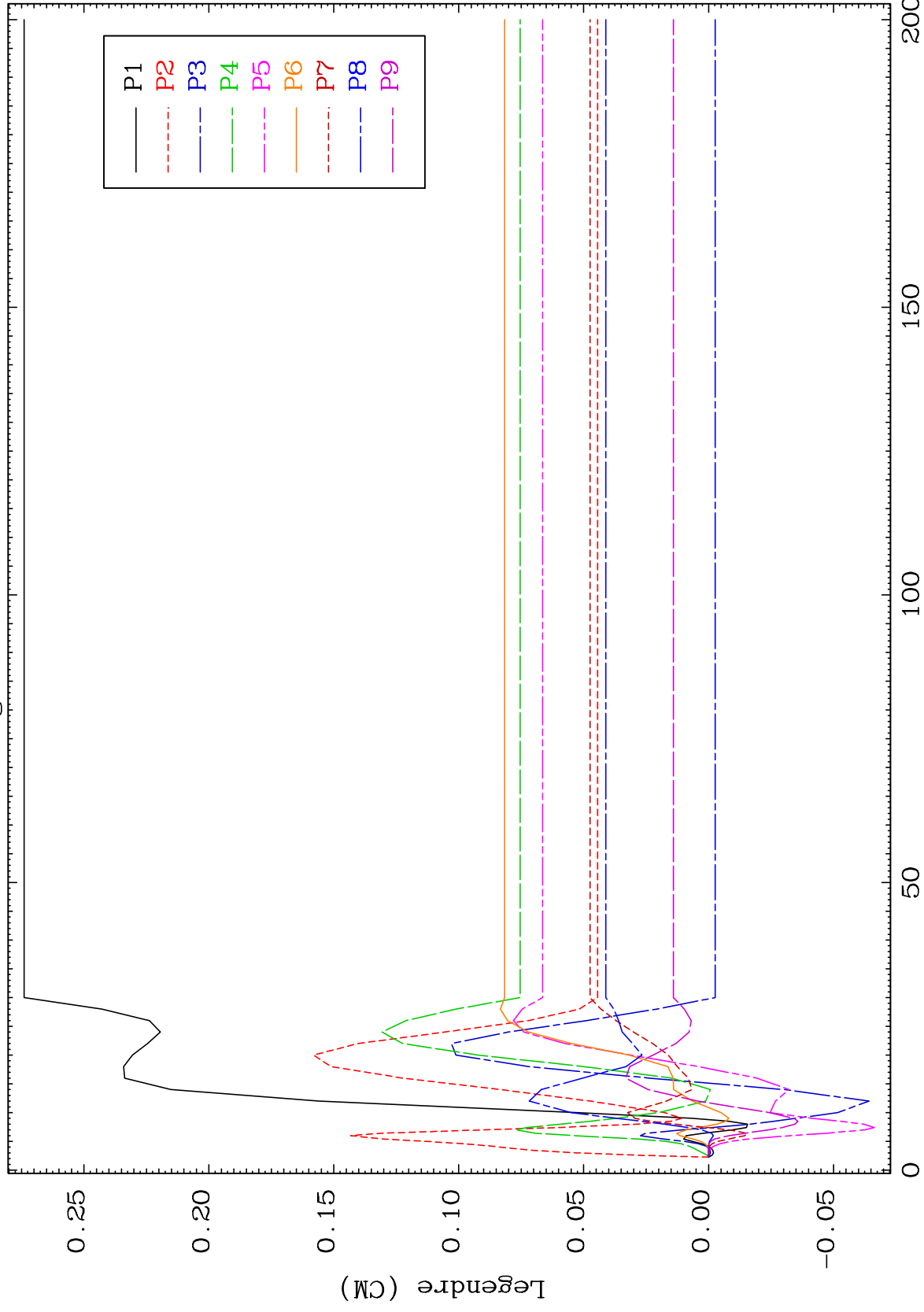




MAT 6001

MT= 66 (n,n') Level
Legendre Coefficients

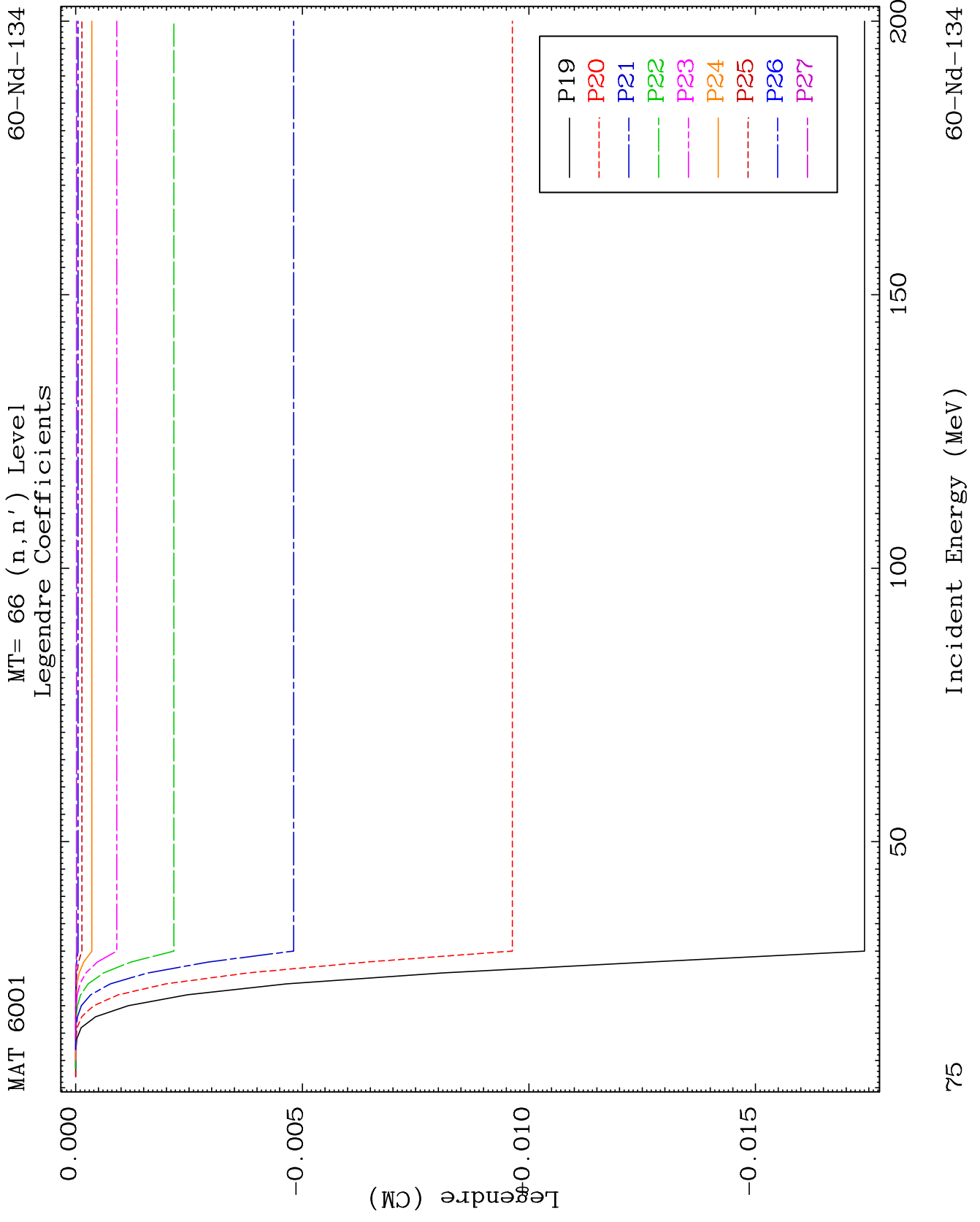
60-Nd-134

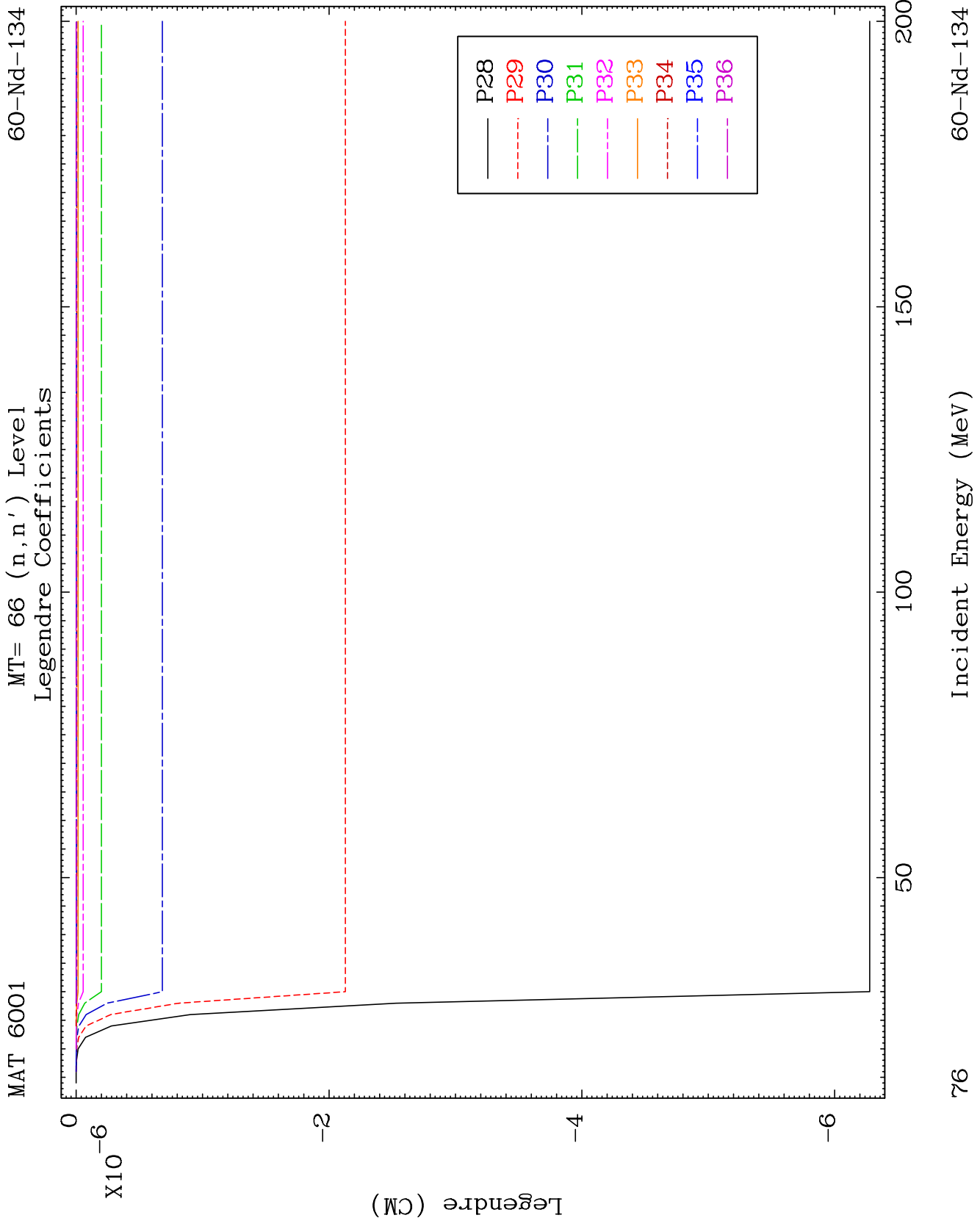


73

Incident Energy (MeV)

60-Nd-134



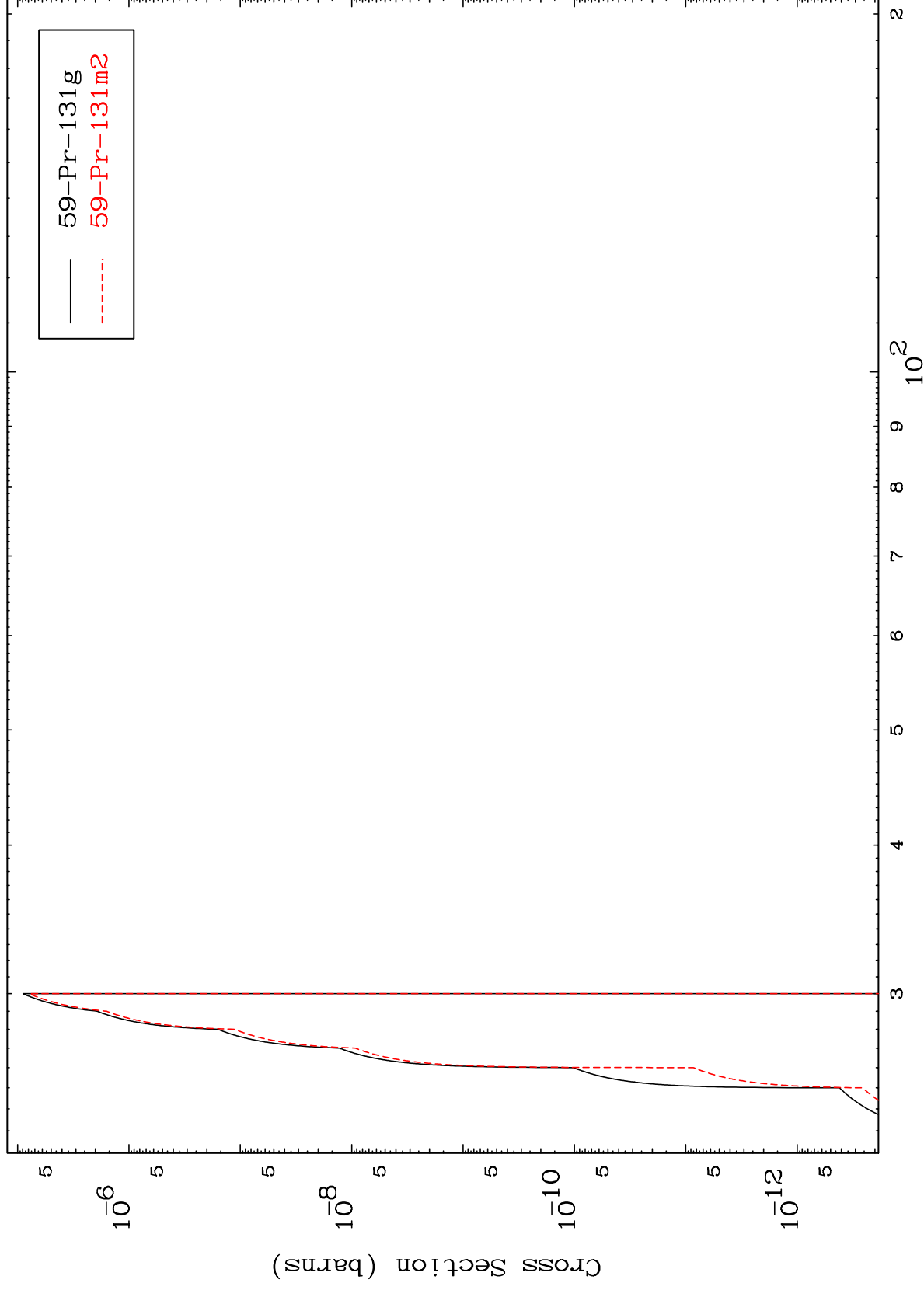


MAT 6001

(n,2n) d

60-Nd-134

Radionuclide Production Cross Section



77

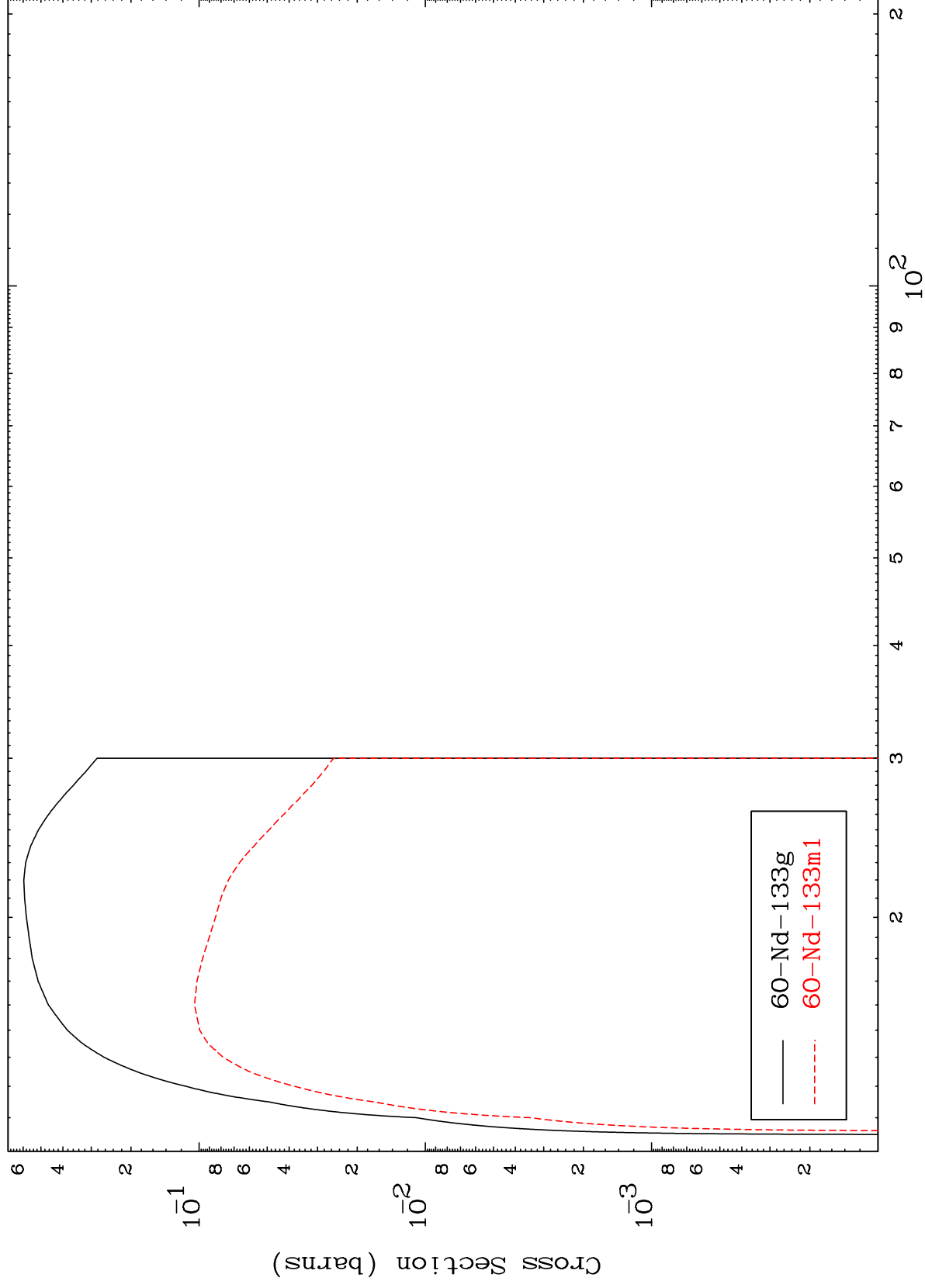
Incident Energy (MeV)

60-Nd-134

MAT 6001

60-Nd-134

(n,2n)
Radionuclide Production Cross Section



78

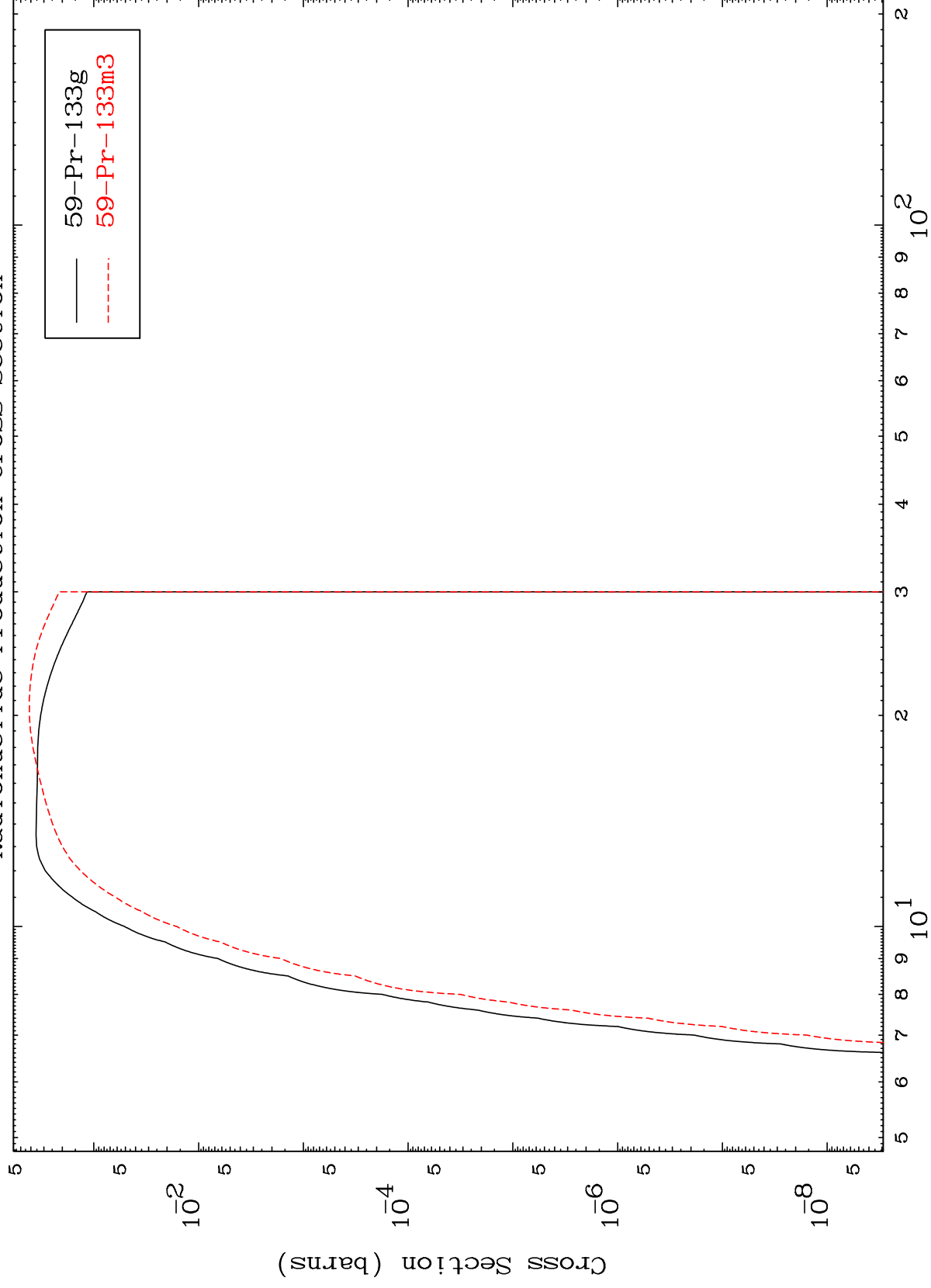
Incident Energy (MeV)

60-Nd-134

MAT 6001

60-Nd-134

(n,n') p
Radionuclide Production Cross Section



79

Incident Energy (MeV)

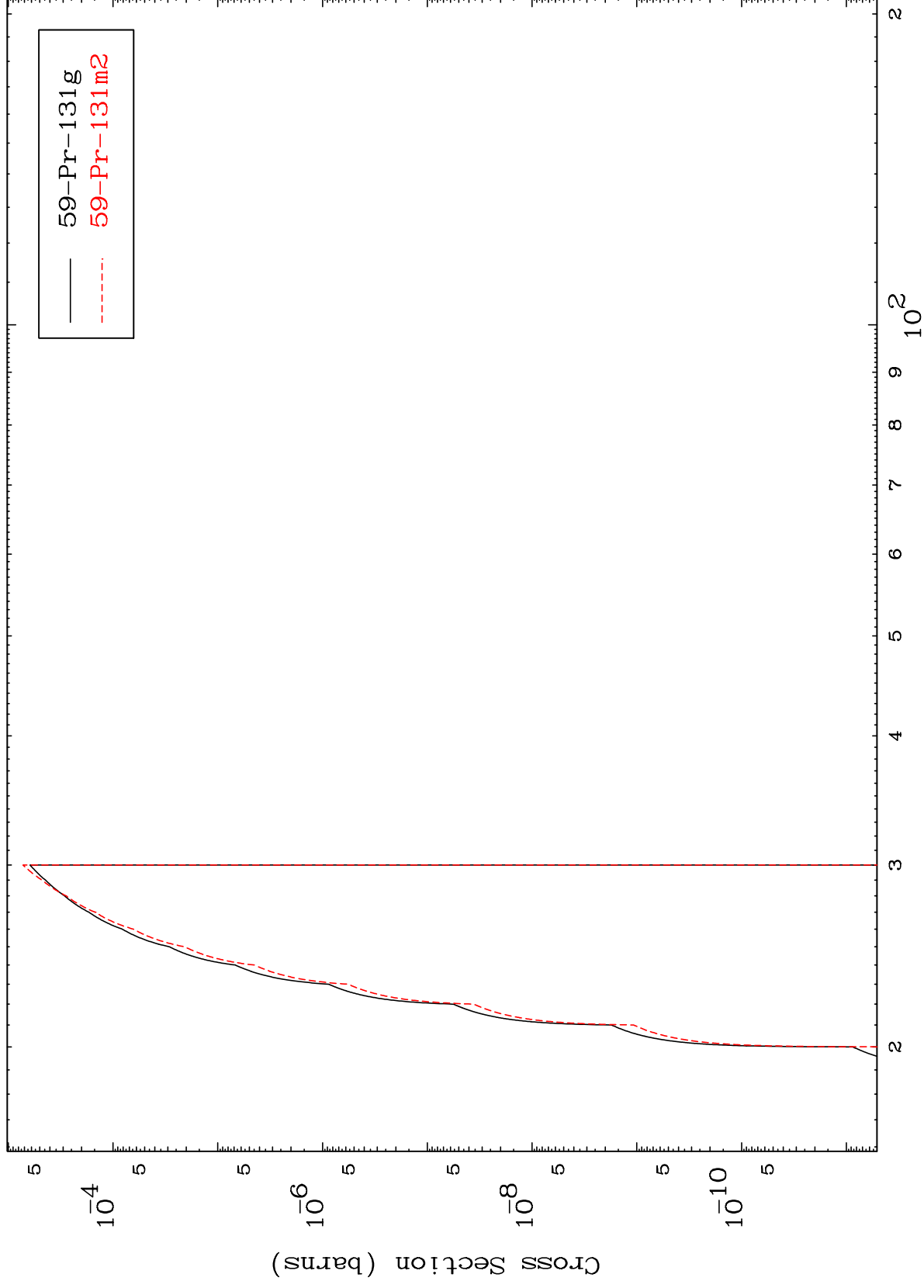
60-Nd-134

MAT 6001

(n,n') t

60-Nd-134

Radionuclide Production Cross Section



80

Incident Energy (MeV)

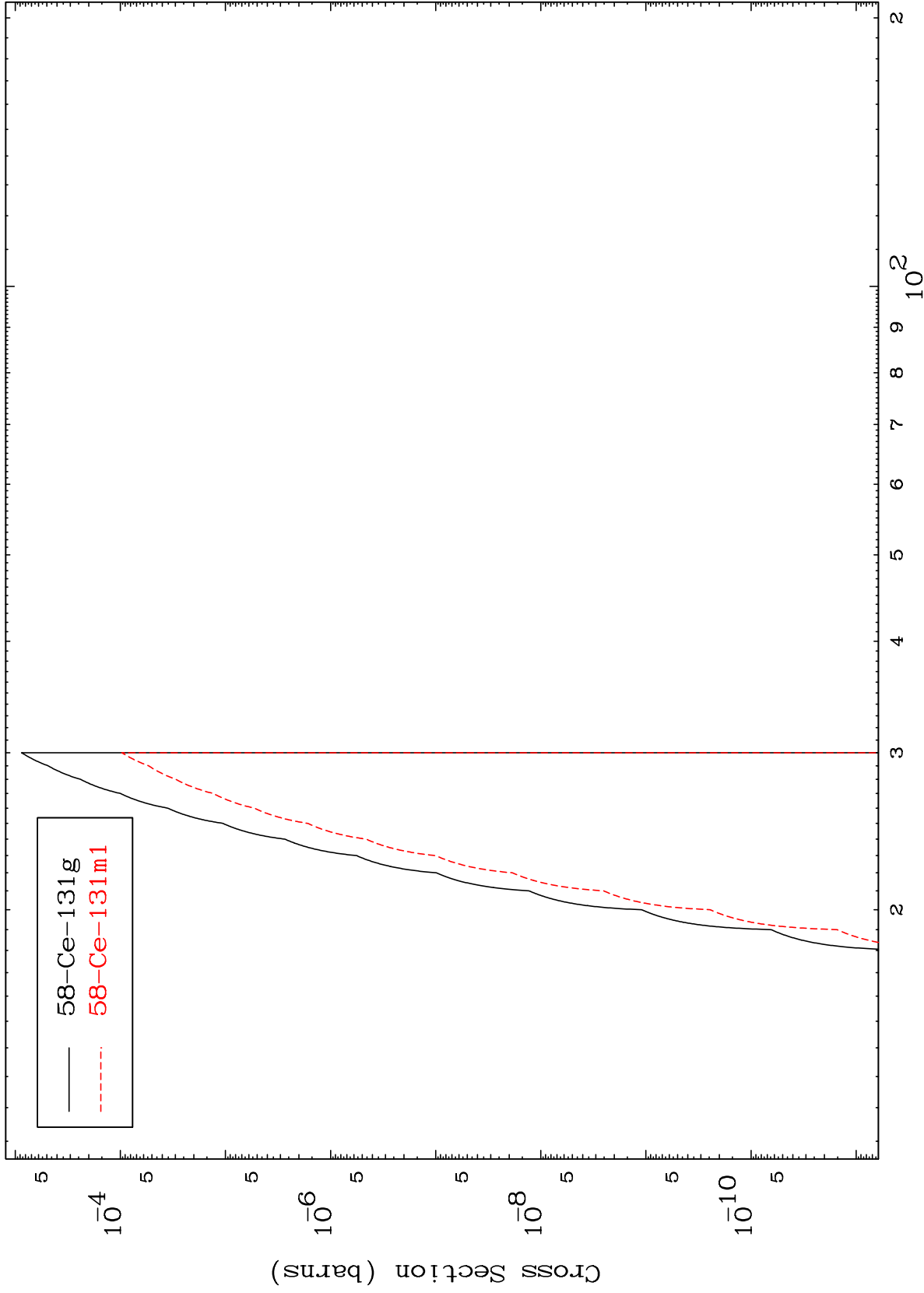
60-Nd-134

MAT 6001

(n,n') He-3

60-Nd-134

Radionuclide Production Cross Section



81

Incident Energy (MeV)

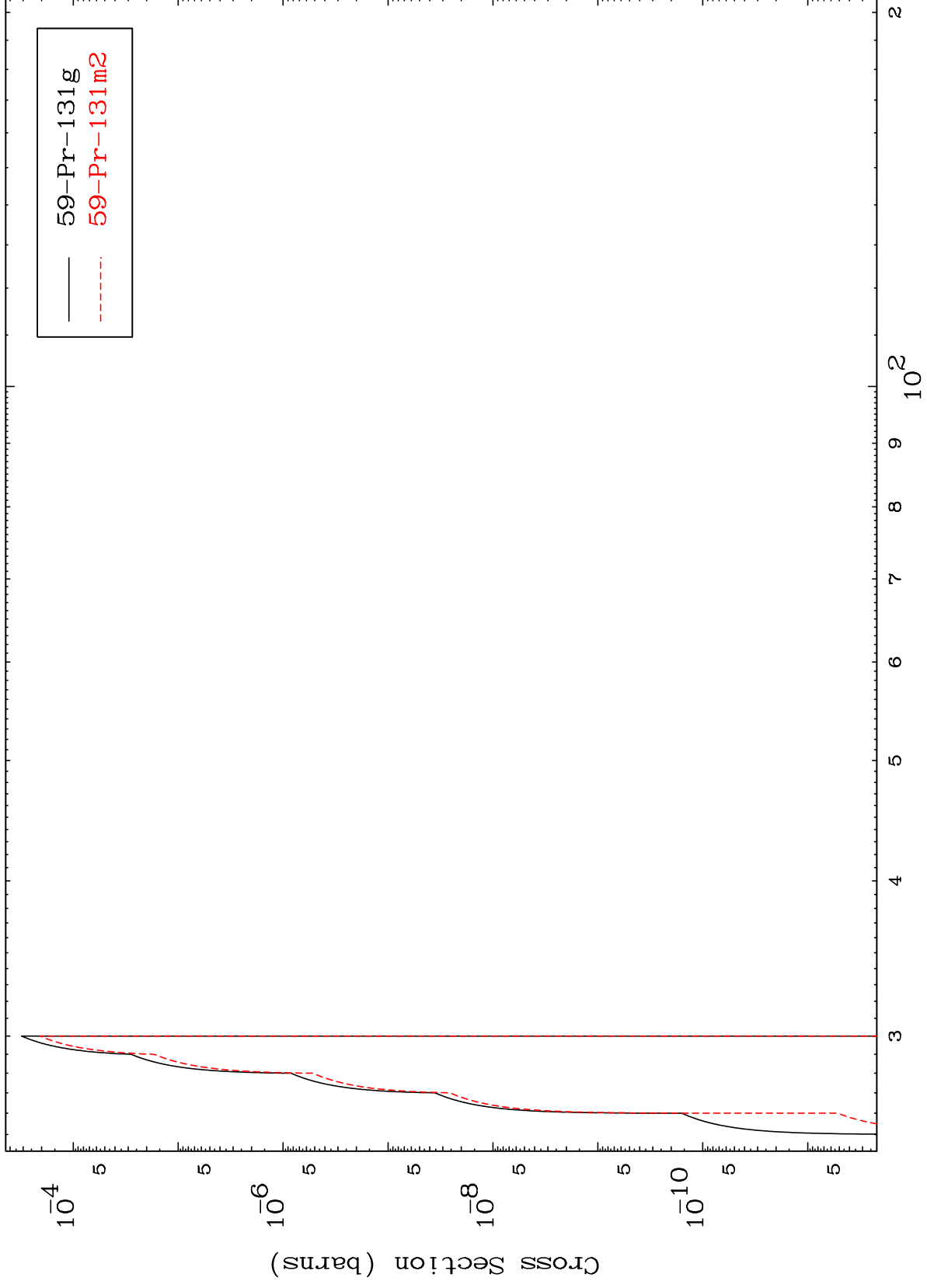
60-Nd-134

MAT 6001

(n,3n) p

60-Nd-134

Radionuclide Production Cross Section



82

Incident Energy (MeV)

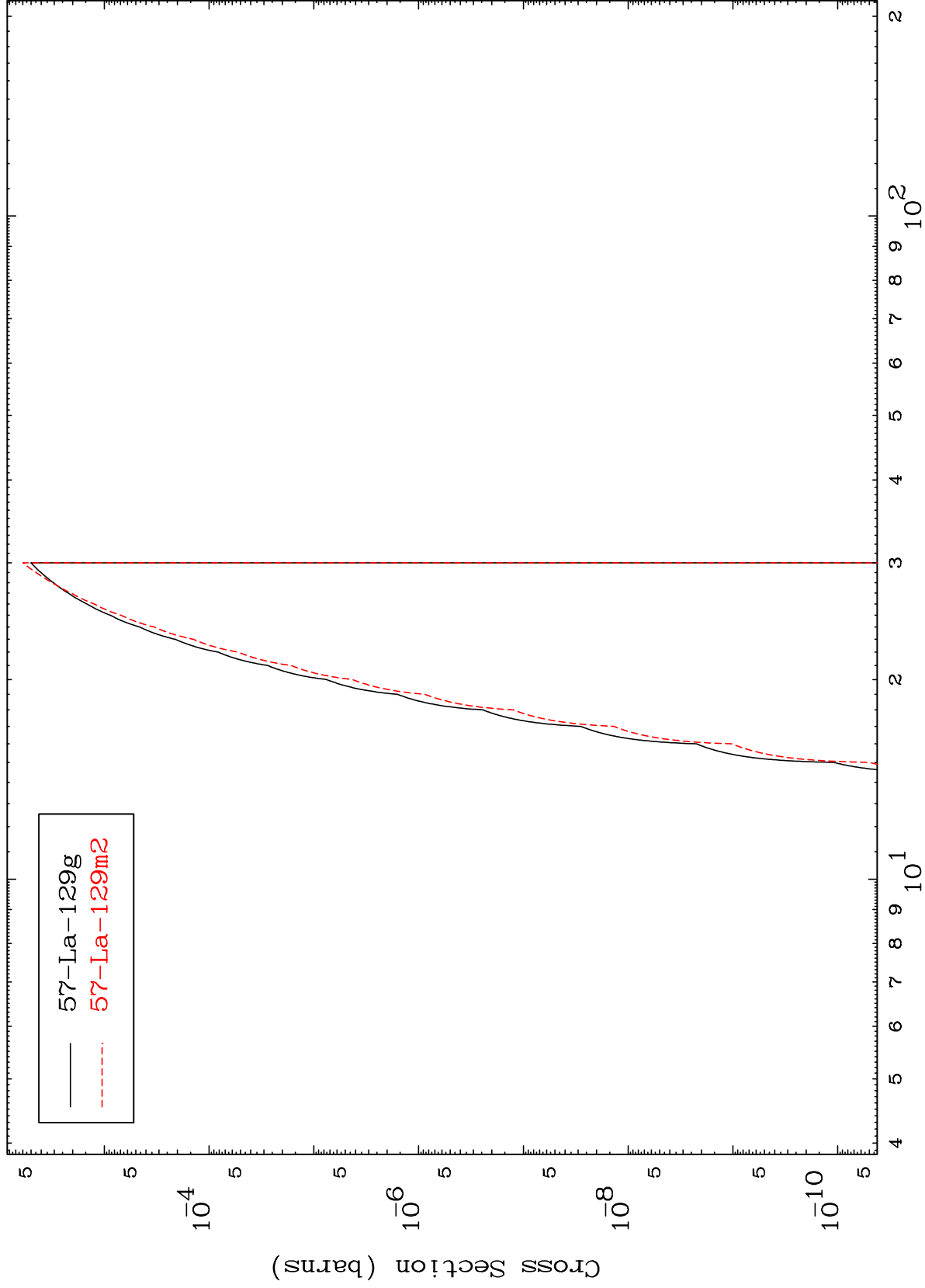
60-Nd-134

MAT 6001

(n,n') p α

60-Nd-134

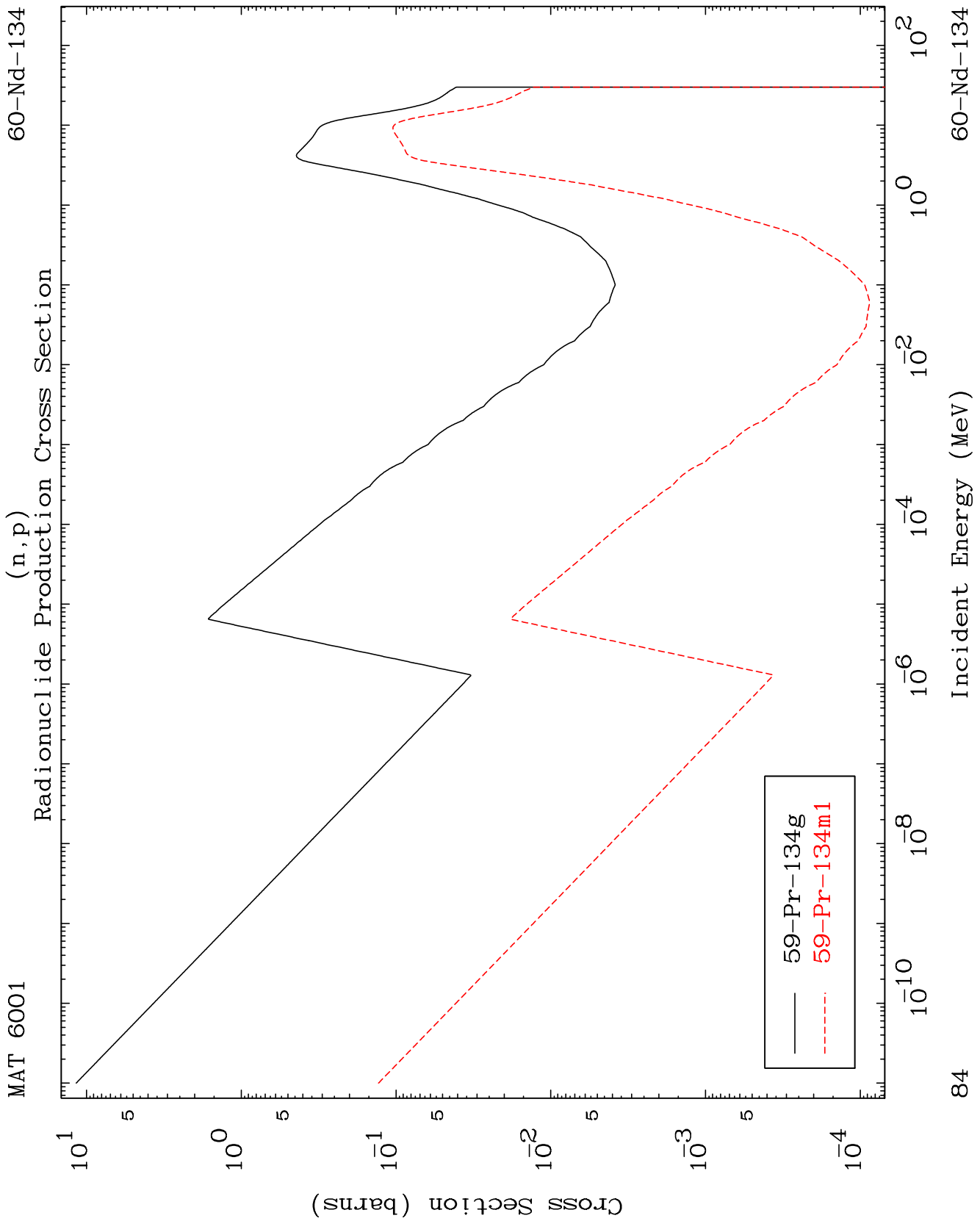
Radionuclide Production Cross Section



83

Incident Energy (MeV)

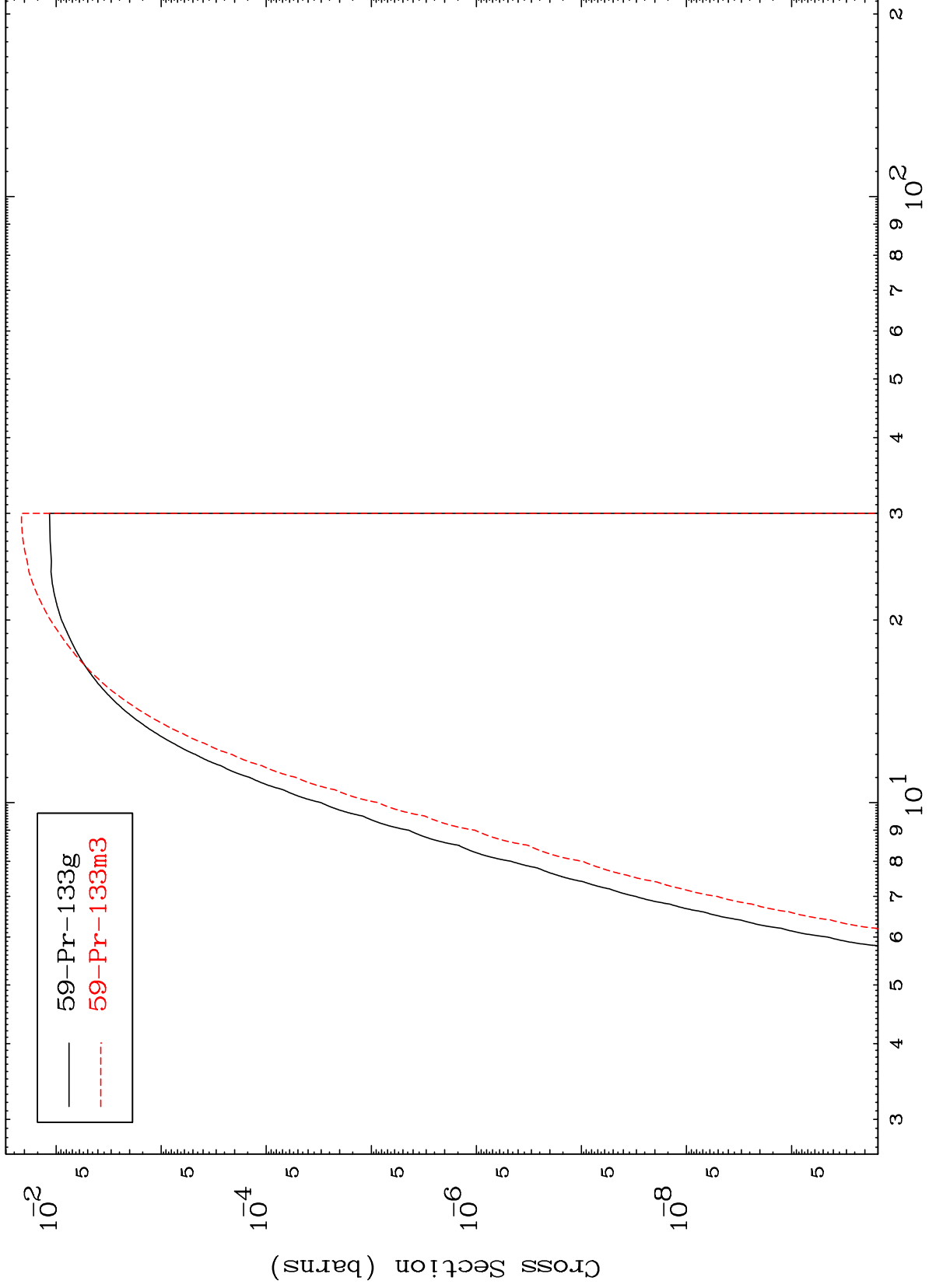
60-Nd-134



MAT 6001

$^{60}\text{Nd-134}$

(n,d)
Radionuclide Production Cross Section



— 59-Pr-133g
- - - 59-Pr-133m3

85

Incident Energy (MeV)

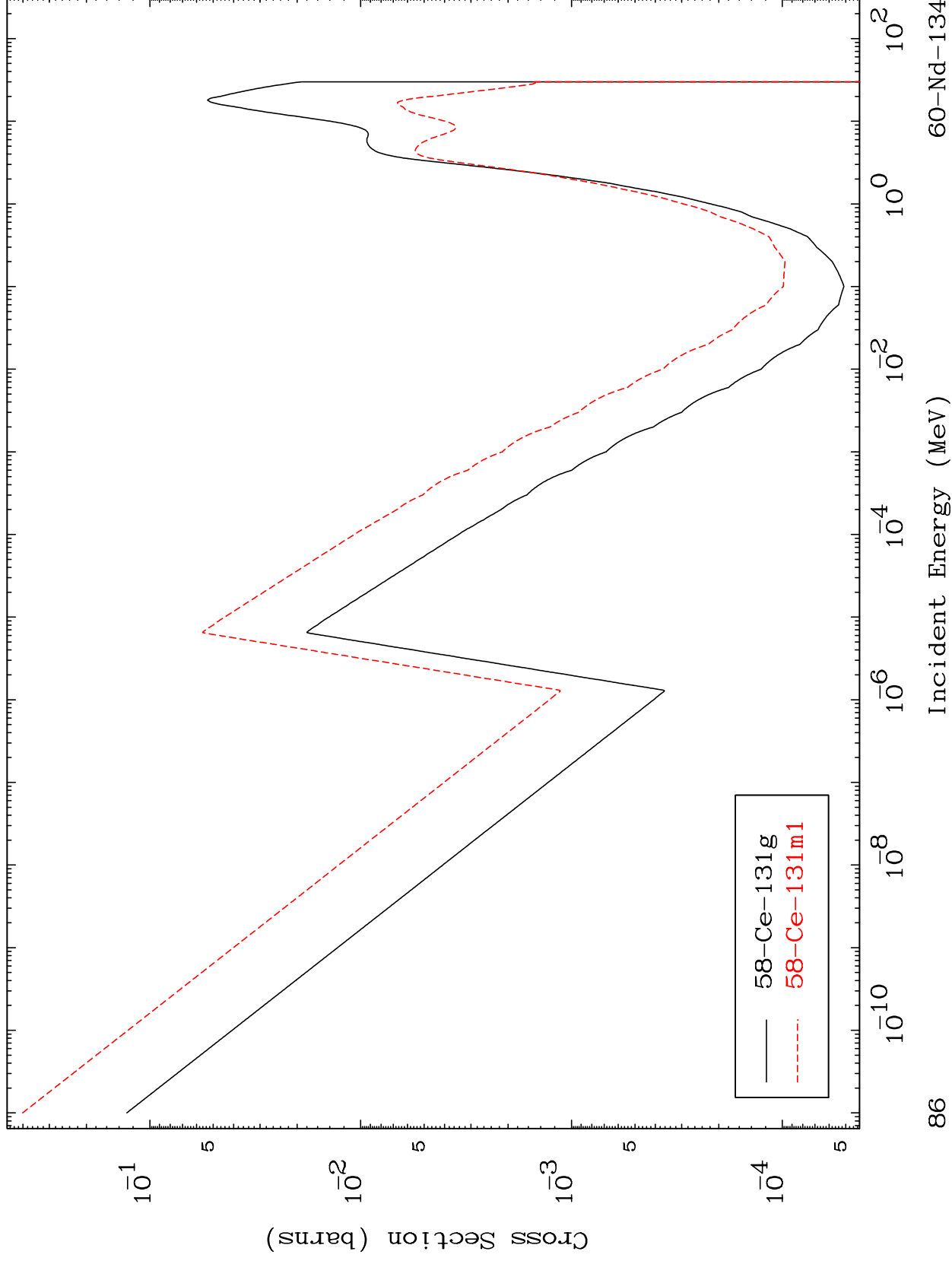
$^{60}\text{Nd-134}$

MAT 6001

(n, α)

60-Nd-134

Radionuclide Production Cross Section

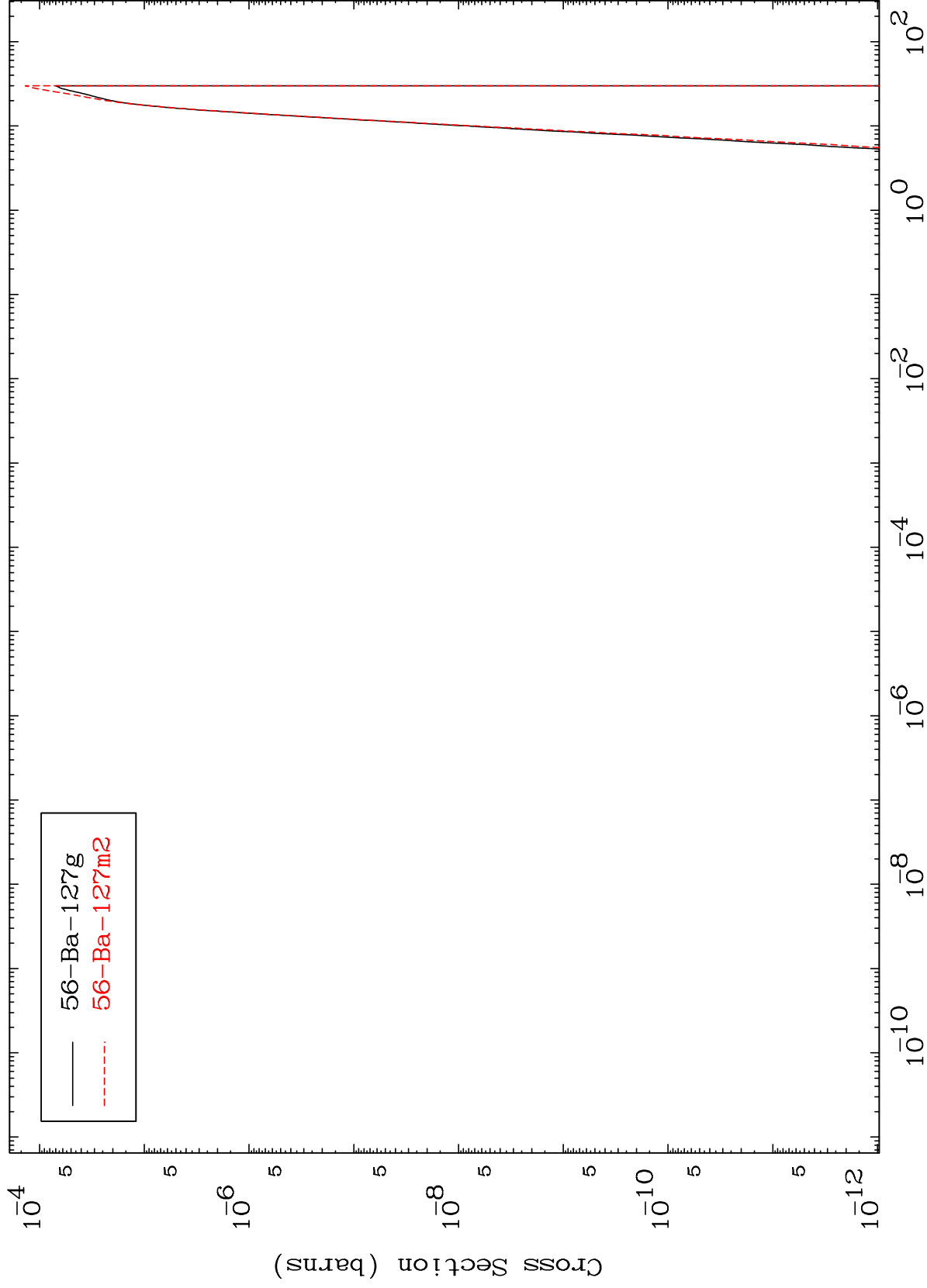


86

MAT 6001

60-Nd-134

(n,2α)
Radionuclide Production Cross Section



87

60-Nd-134

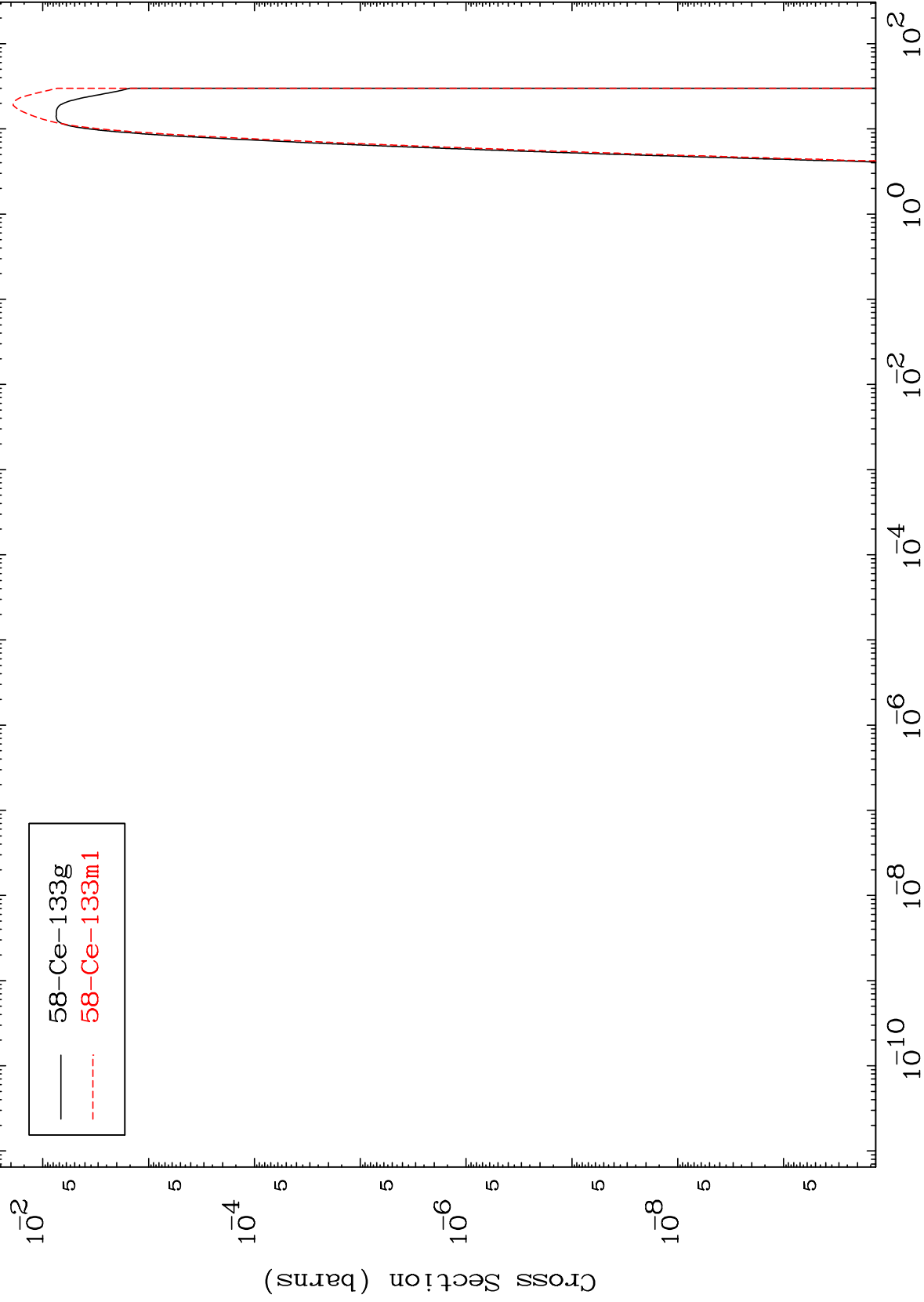
Incident Energy (MeV)

MAT 6001

(n,2p)

60-Nd-134

Radionuclide Production Cross Section



88

Incident Energy (MeV)

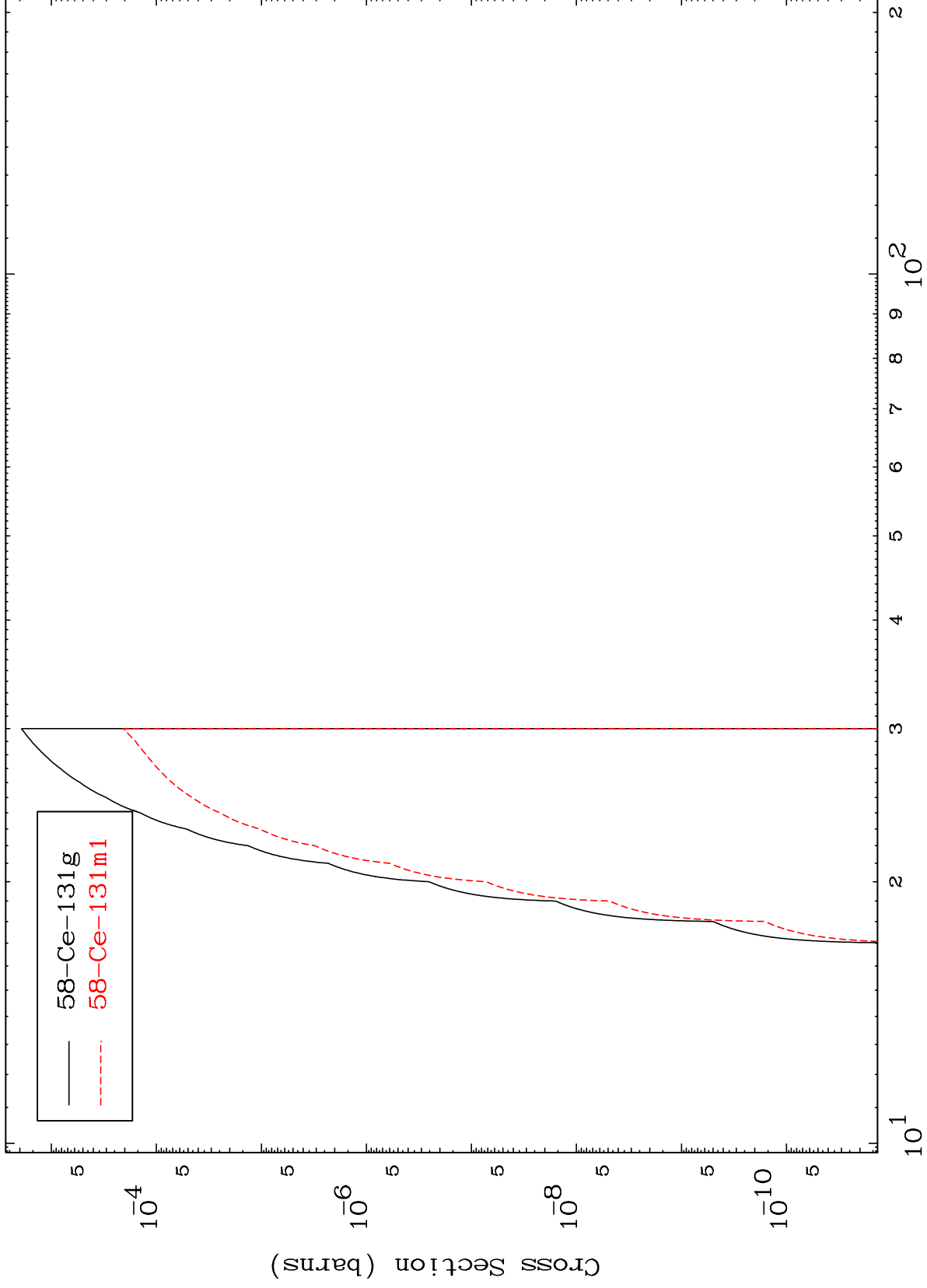
60-Nd-134

MAT 6001

(n,p) t

⁶⁰Nd-134

Radionuclide Production Cross Section



Incident Energy (MeV)

⁶⁰Nd-134

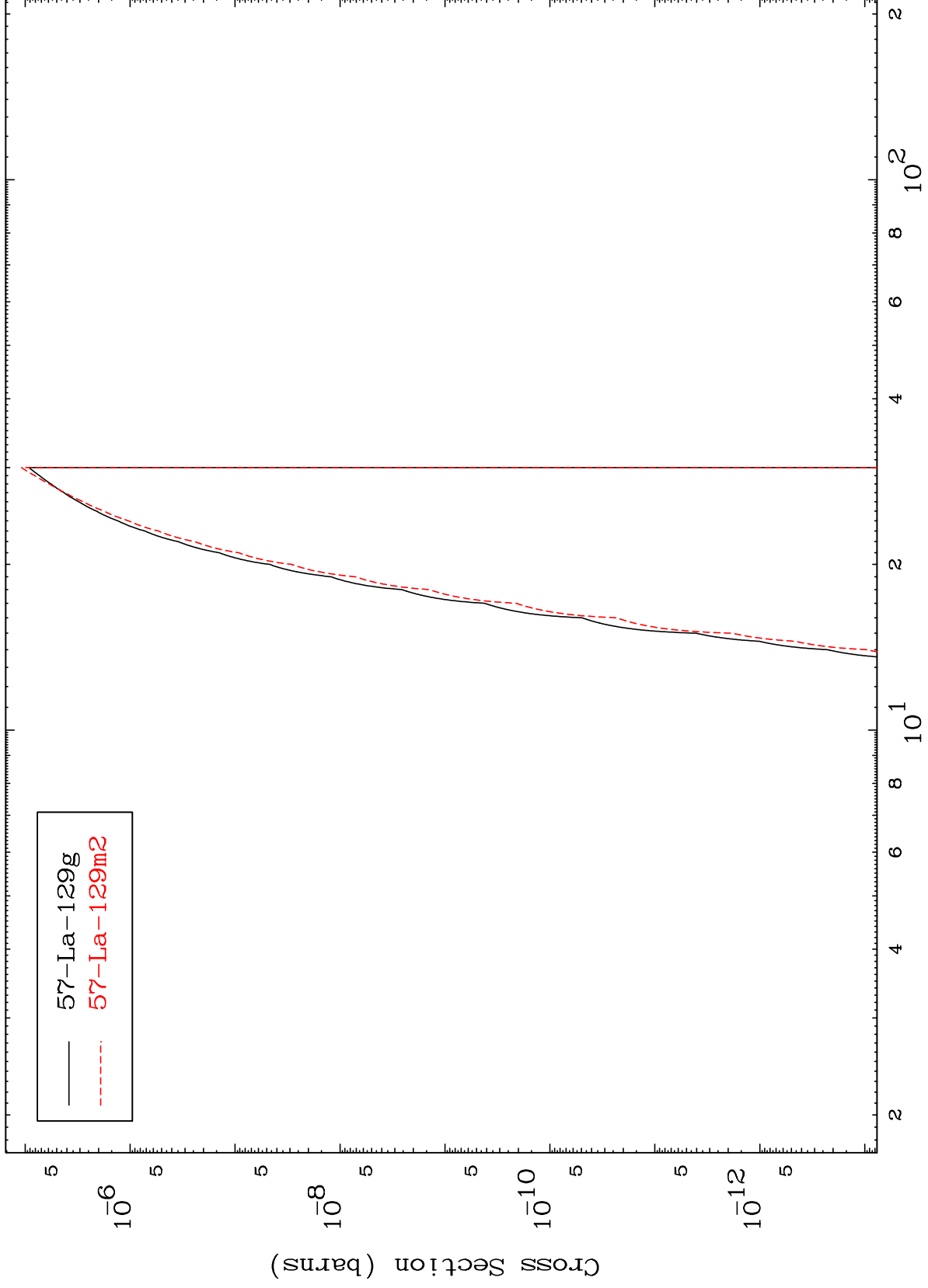
89

MAT 6001

(n,d) α

$^{60}\text{Nd-134}$

Radionuclide Production Cross Section



— $^{57}\text{La-129g}$
- - - $^{57}\text{La-129m2}$

90

Incident Energy (MeV)

$^{60}\text{Nd-134}$