

Program EVALPLOT
(Version 2017-1)

by

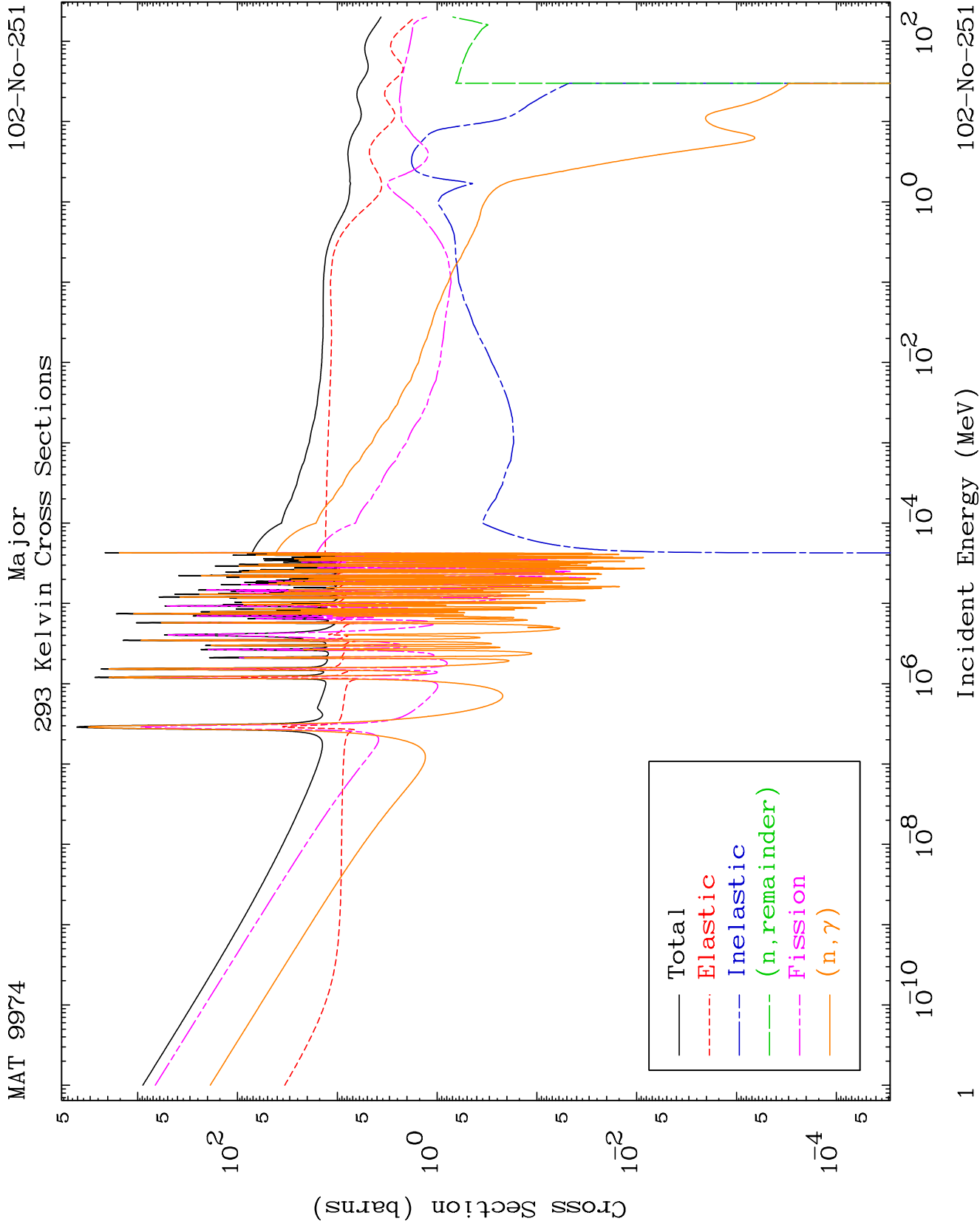
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
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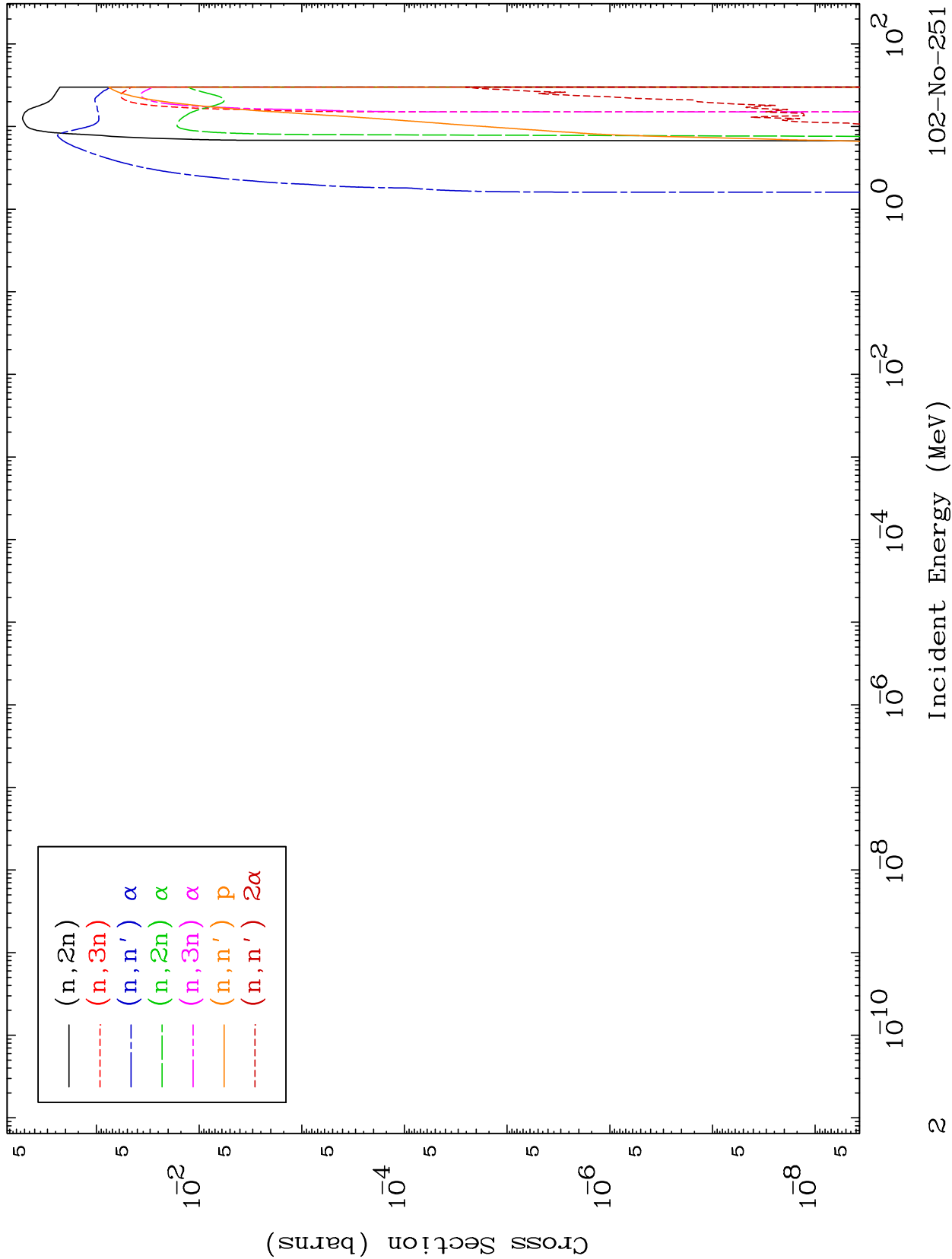
Press Mouse Button to Start



MAT 9974

Neutron Production
293 Kelvin Cross Sections

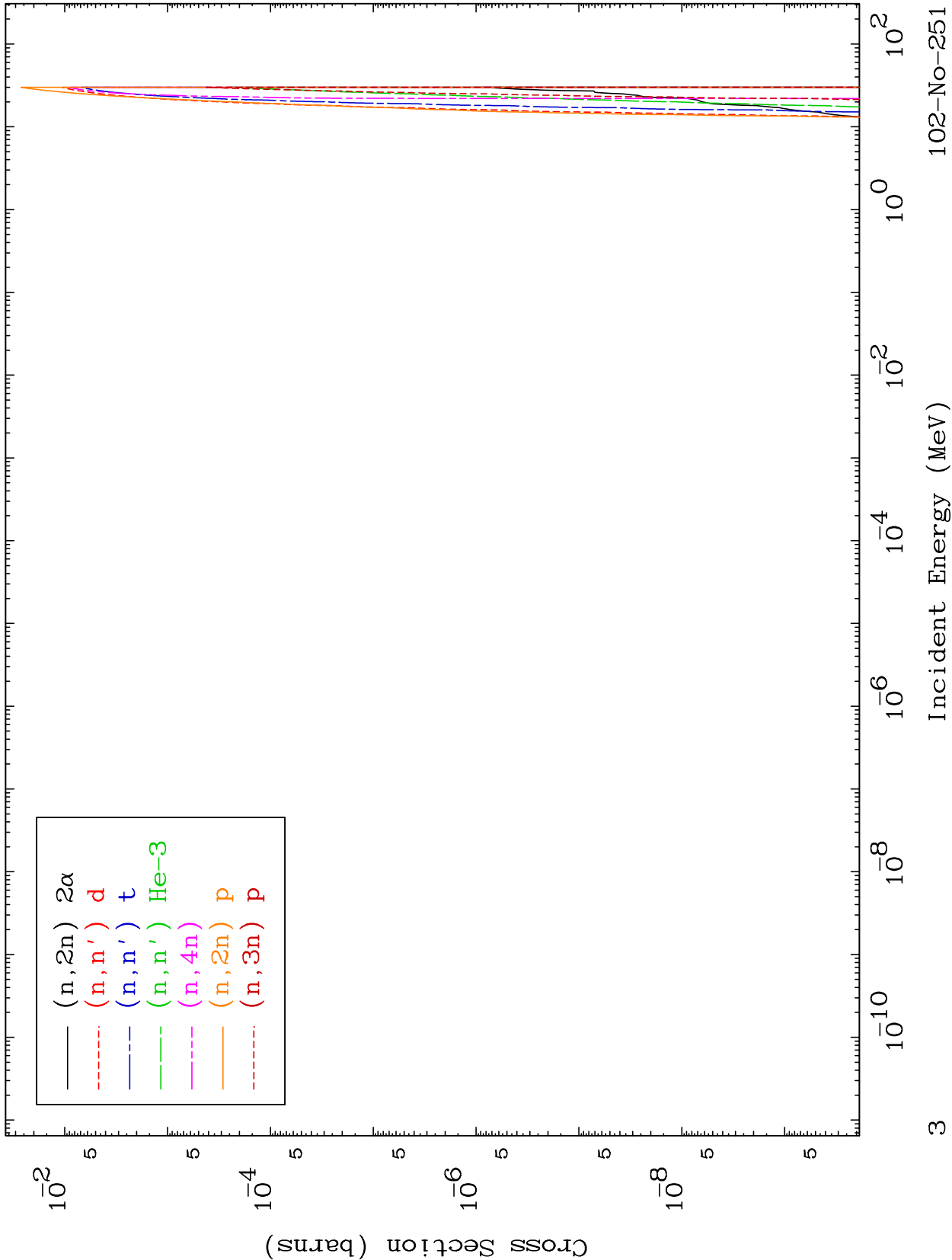
102-No-251



MAT 9974

Neutron Production
293 Kelvin Cross Sections

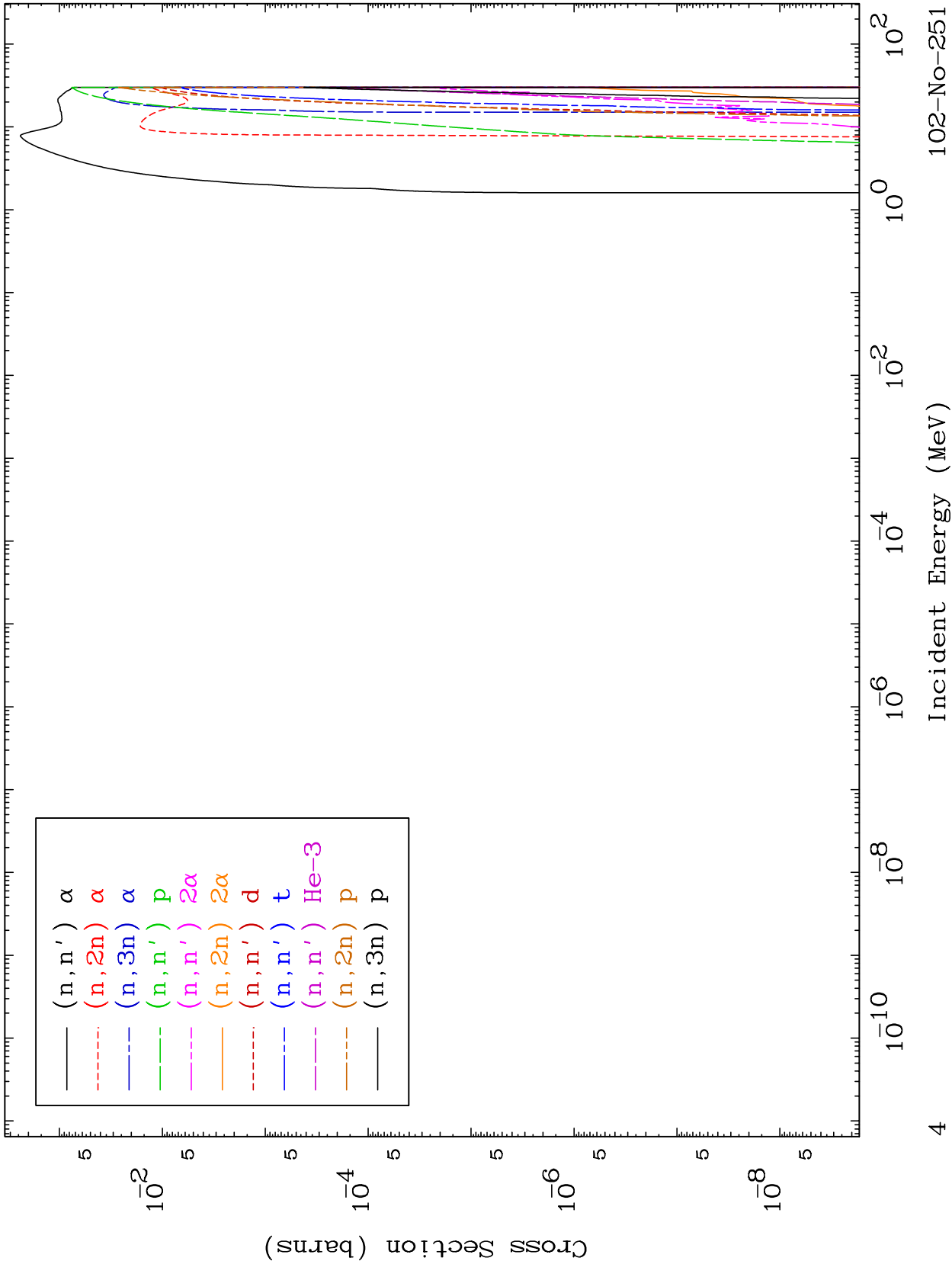
102-No-251

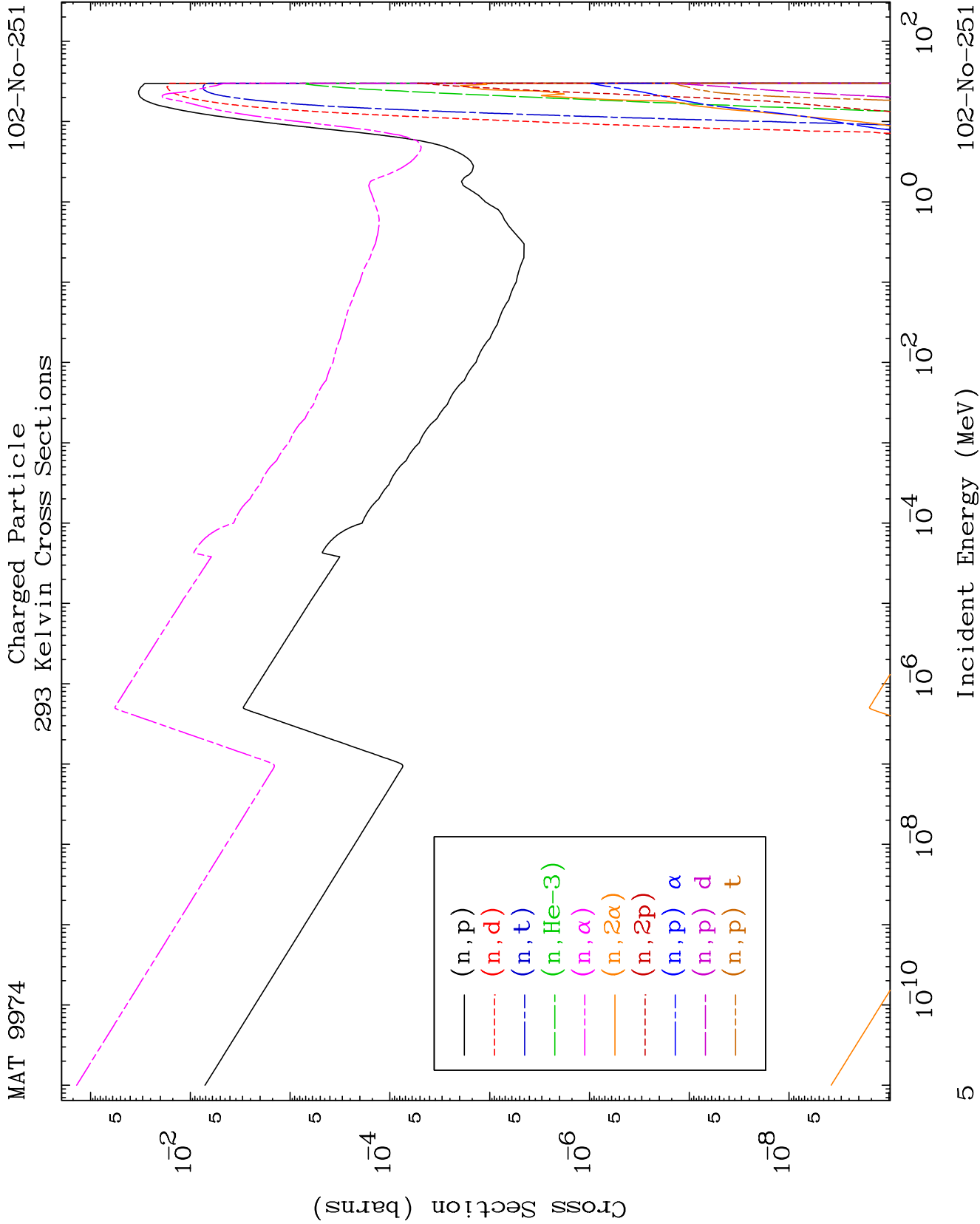


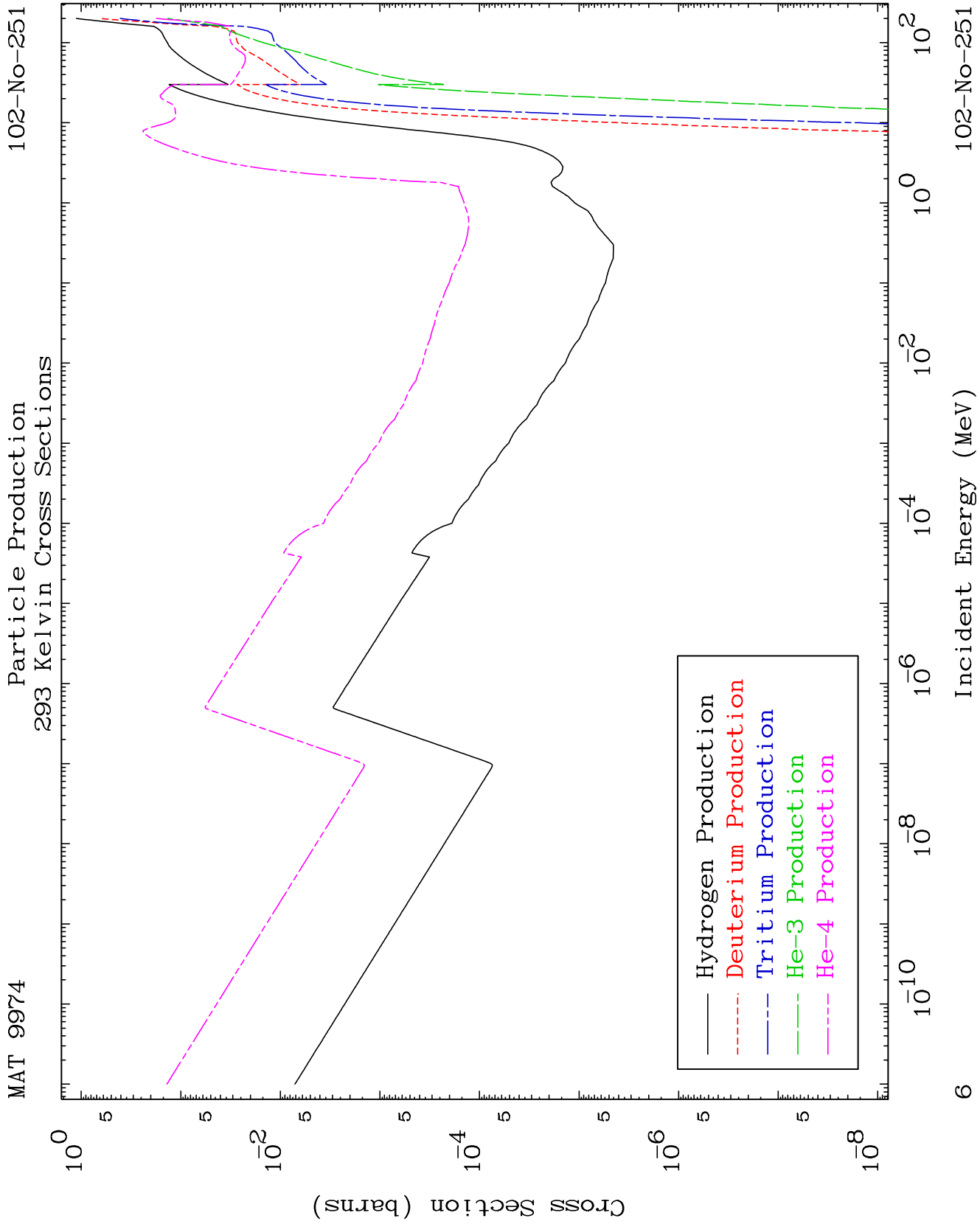
MAT 9974

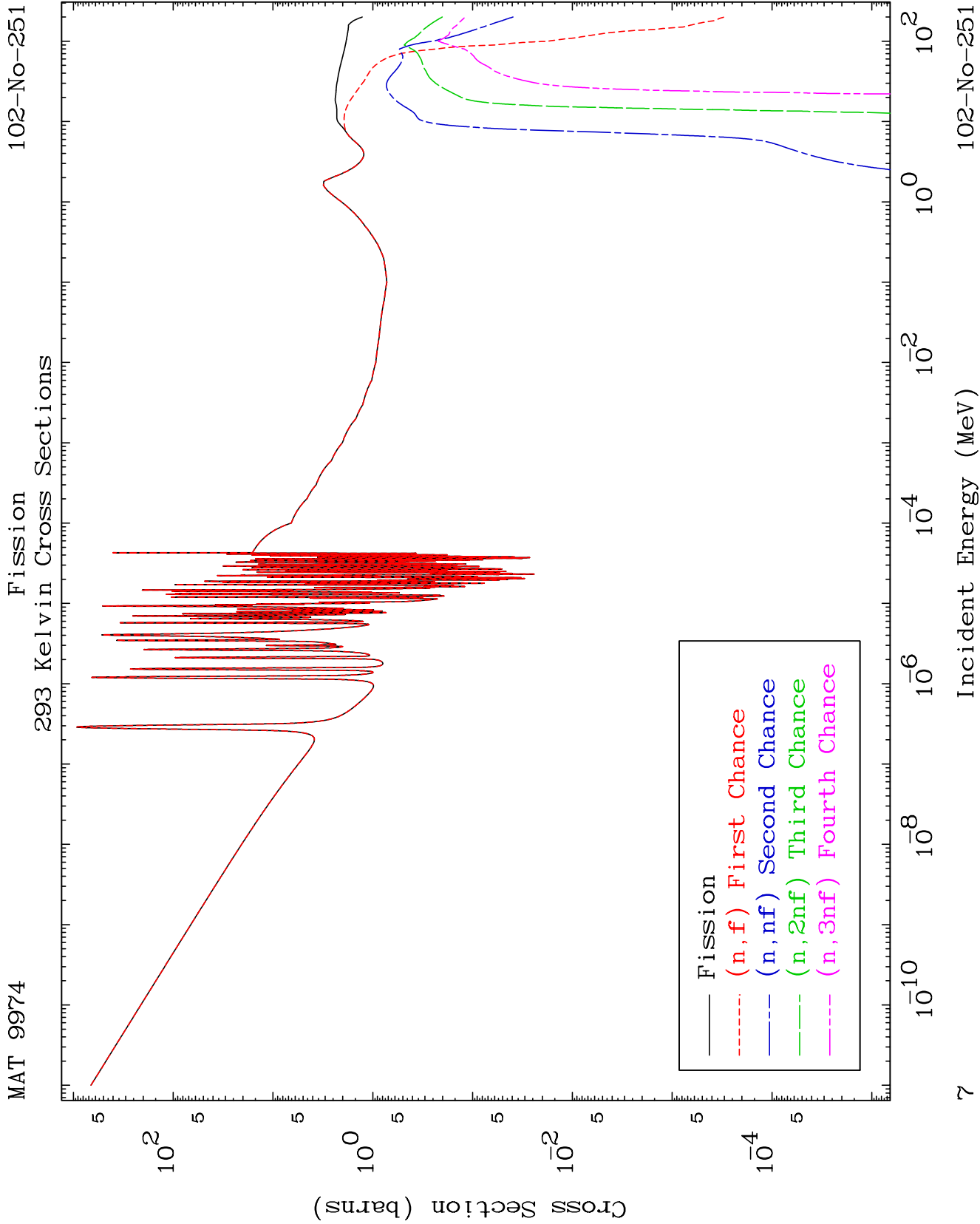
Charged Particle
293 Kelvin Cross Sections

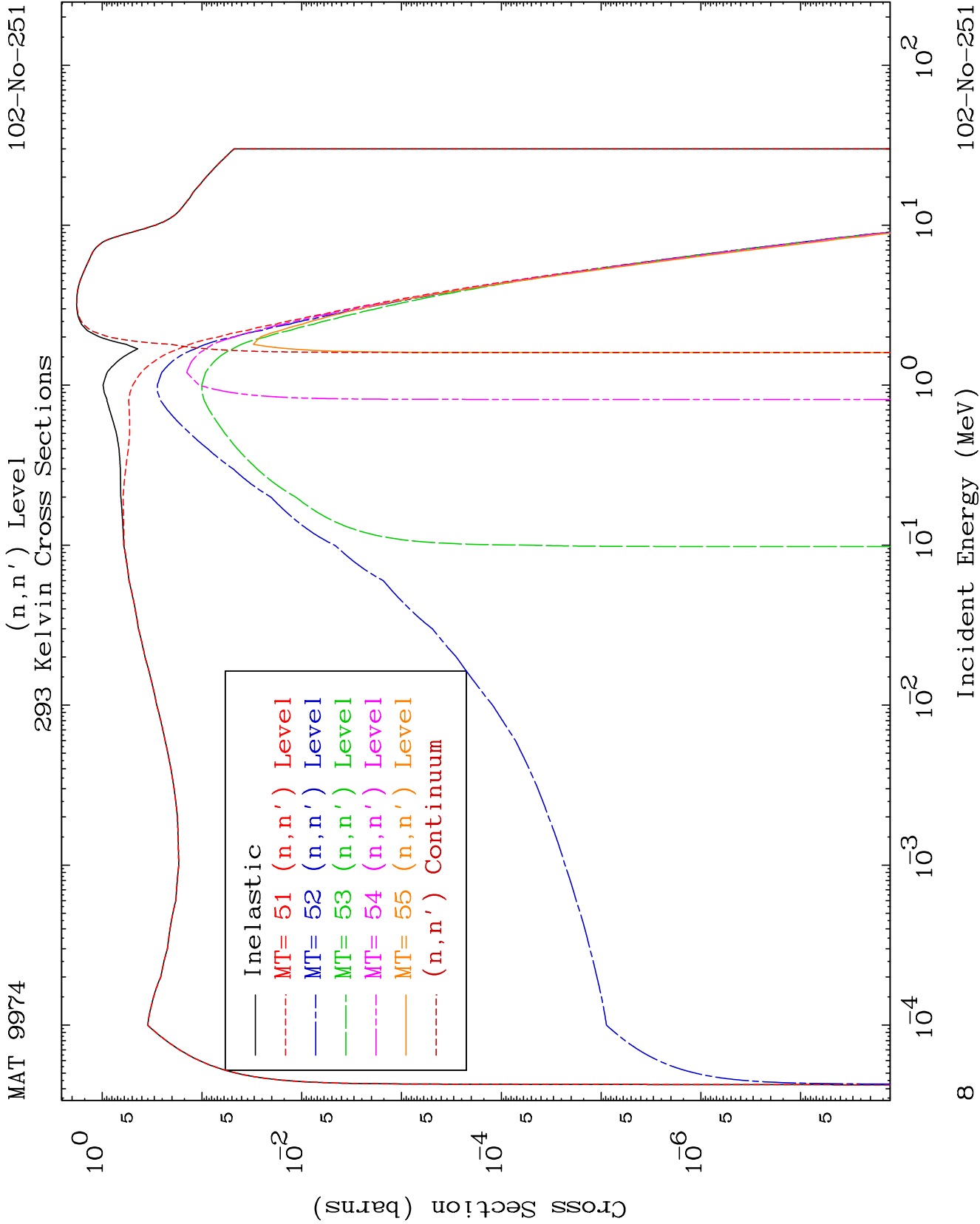
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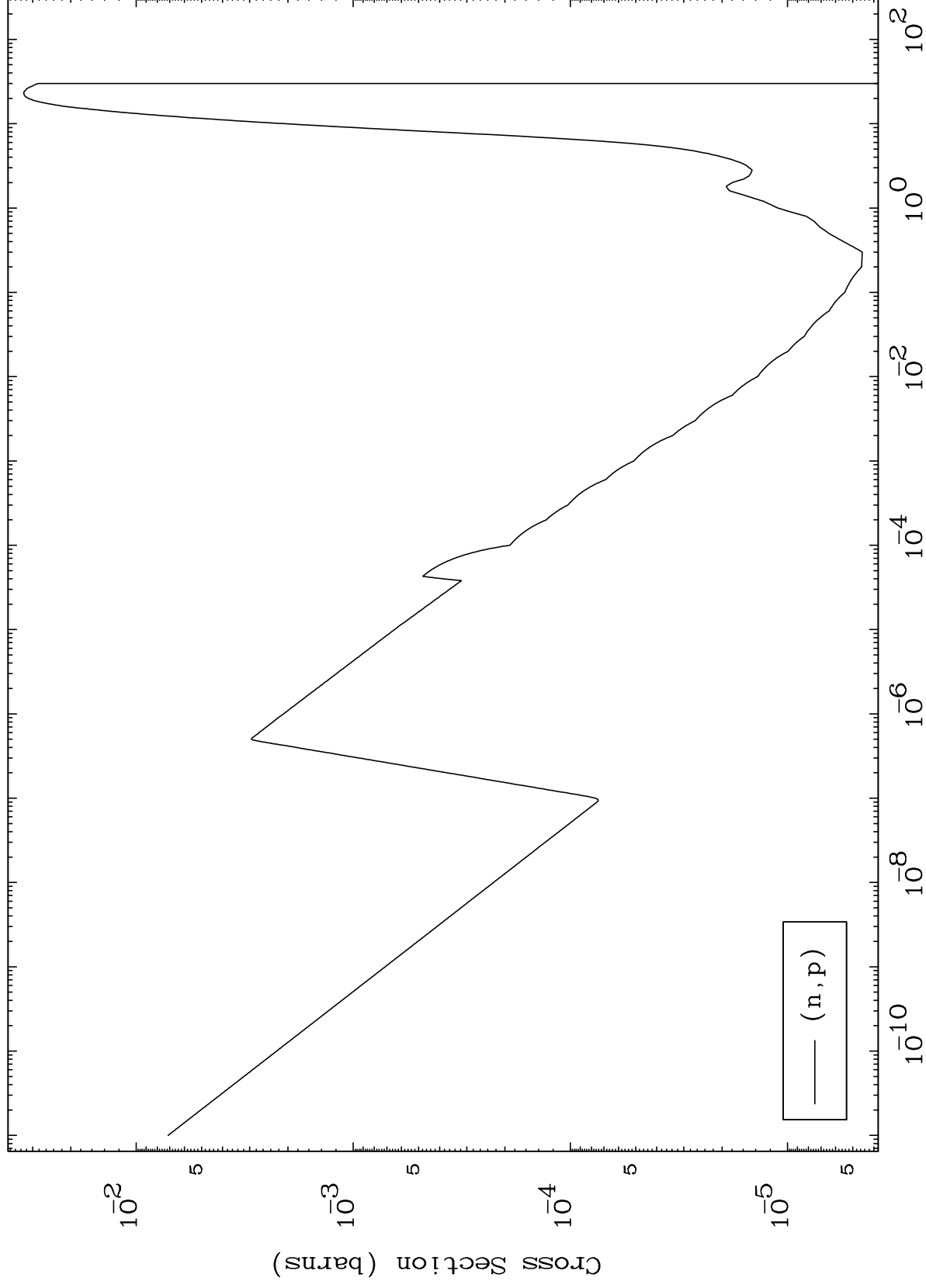


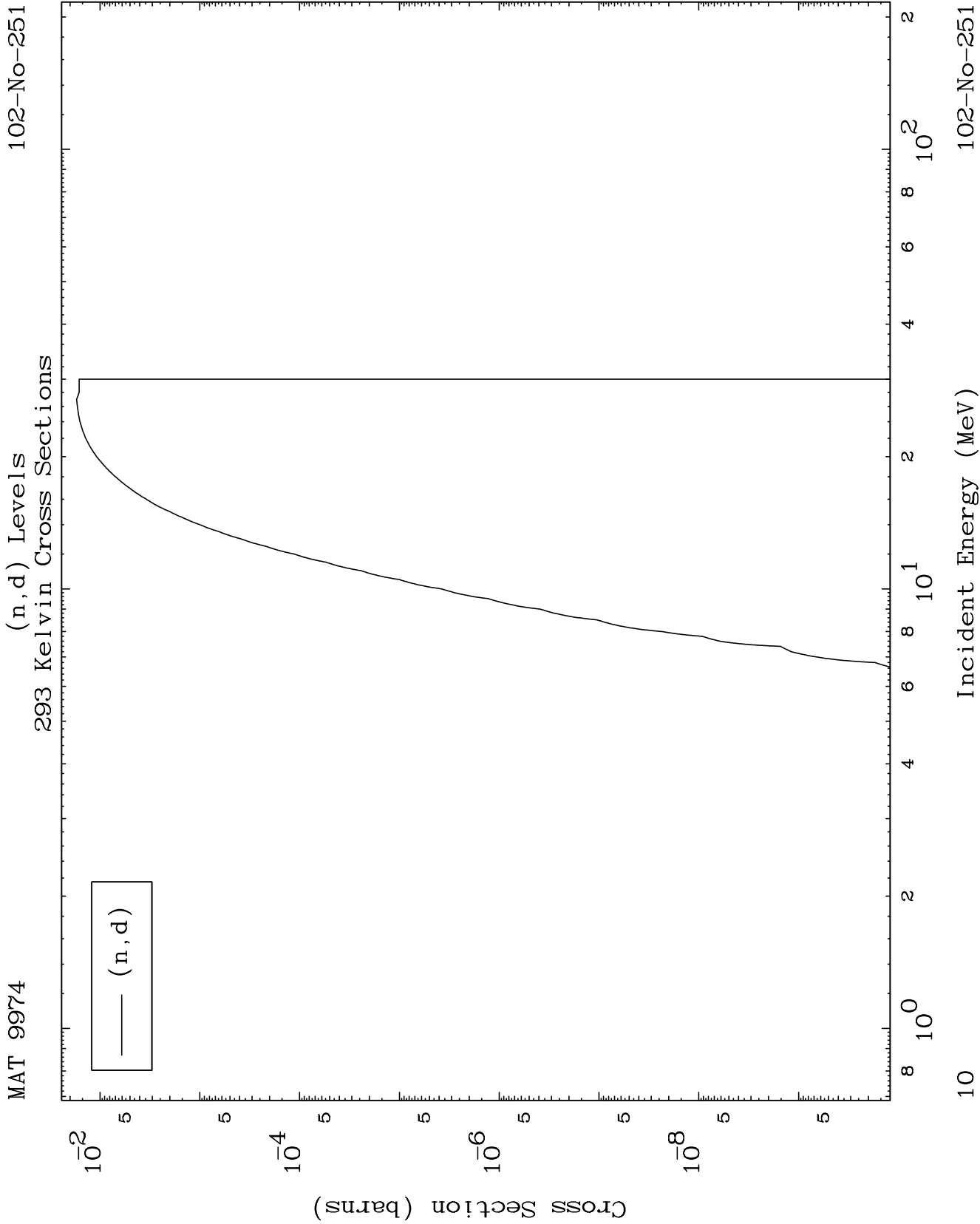


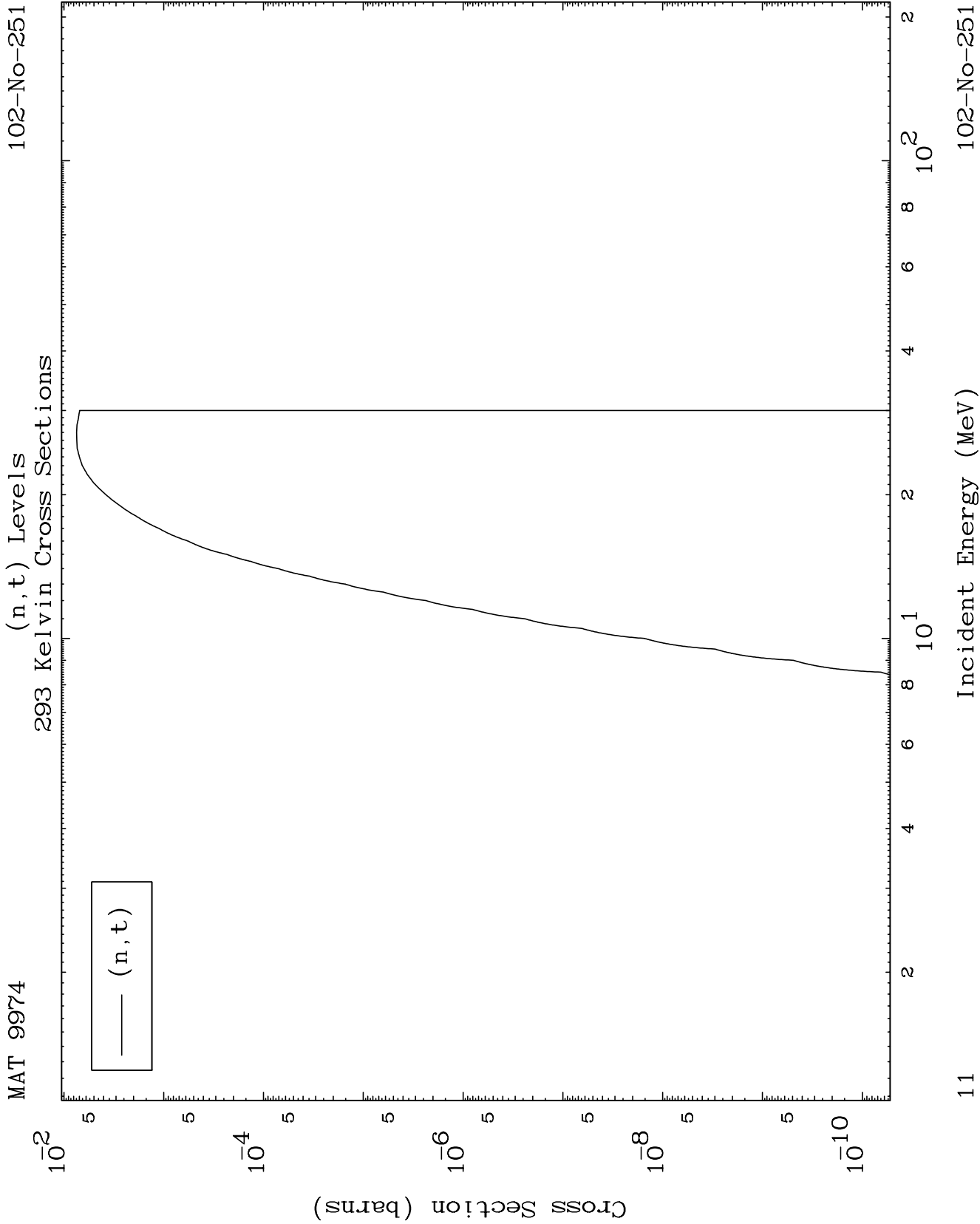
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(n,p) Levels
293 Kelvin Cross Sections

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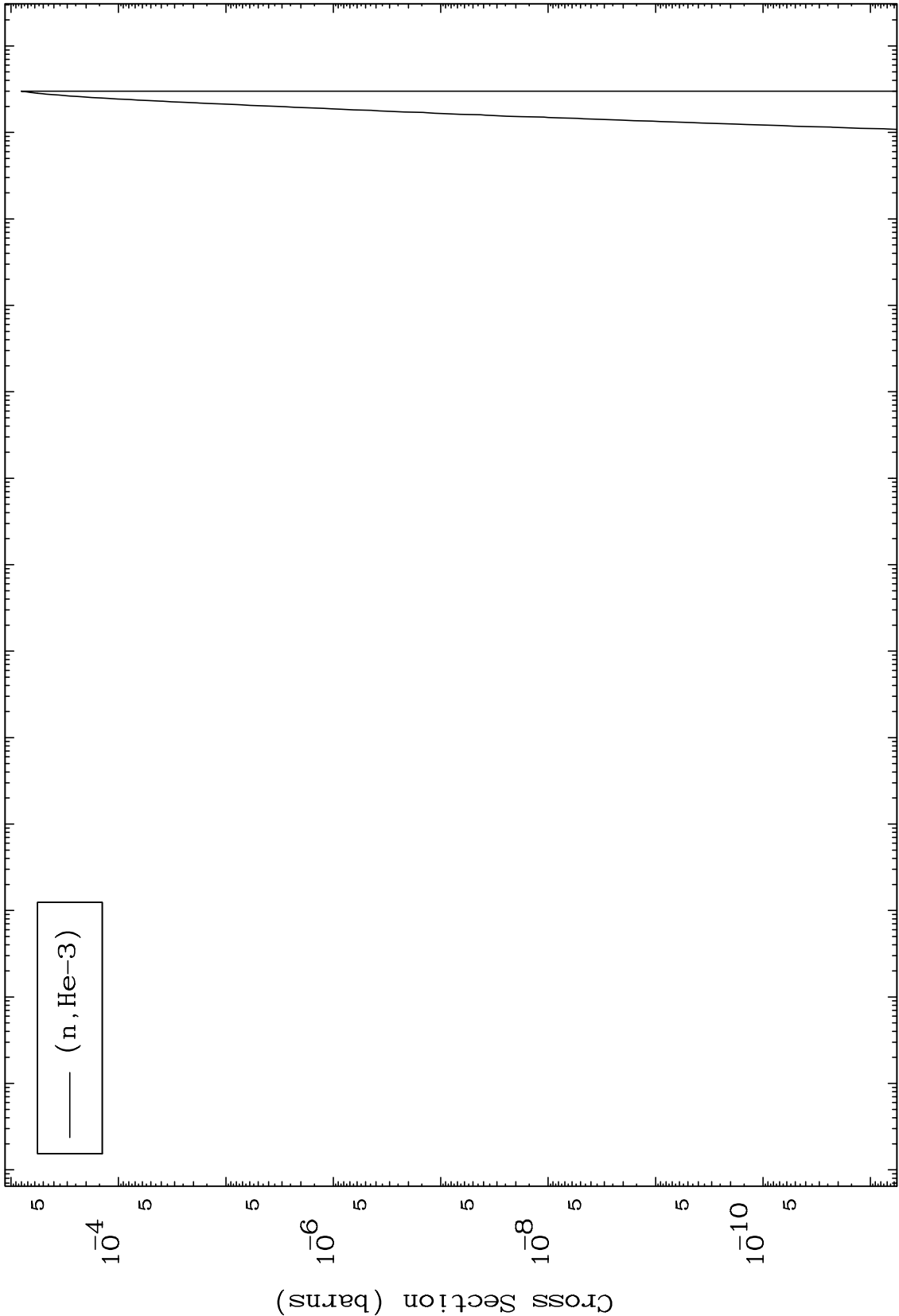




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(n,He3) Levels
293 Kelvin Cross Sections

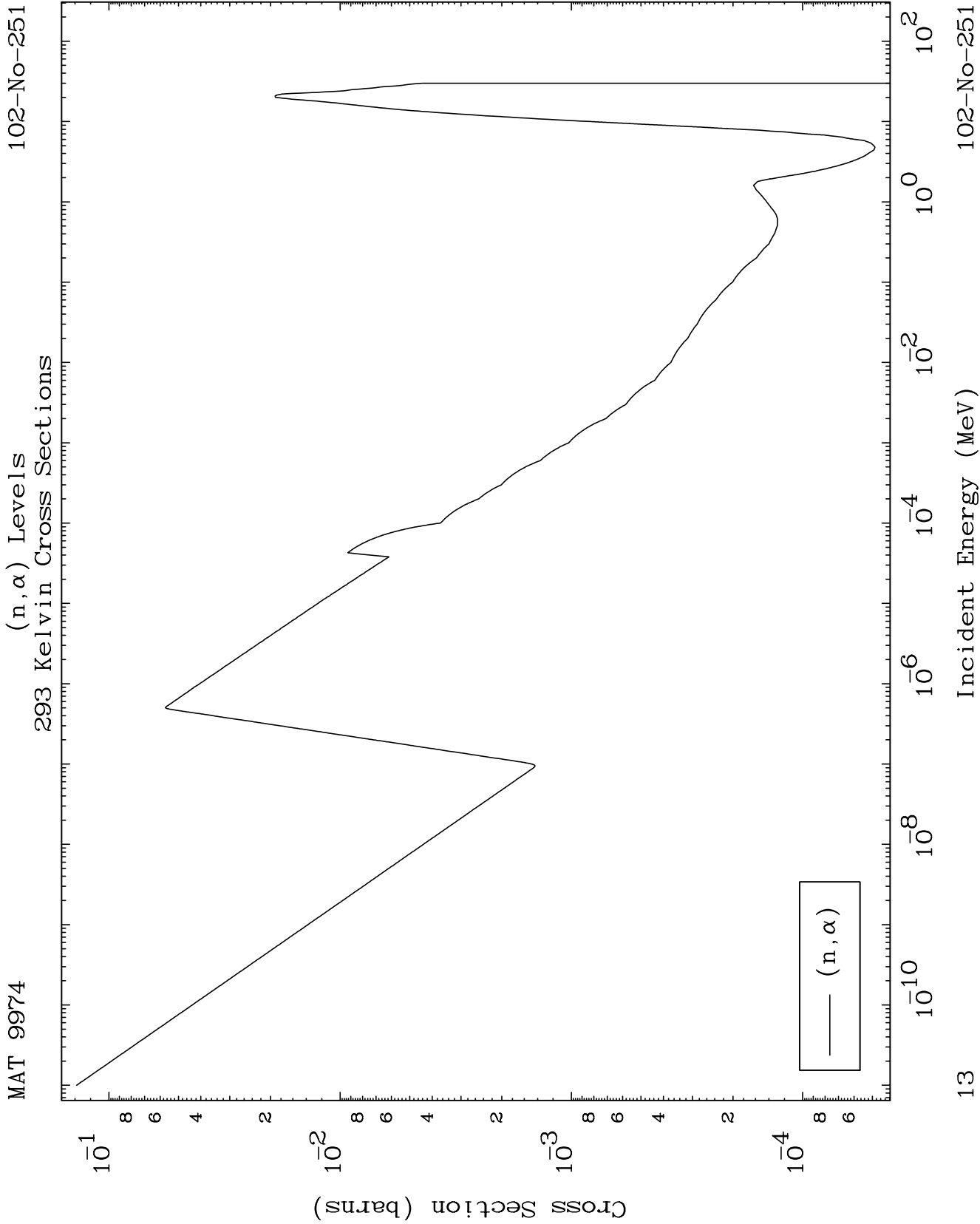
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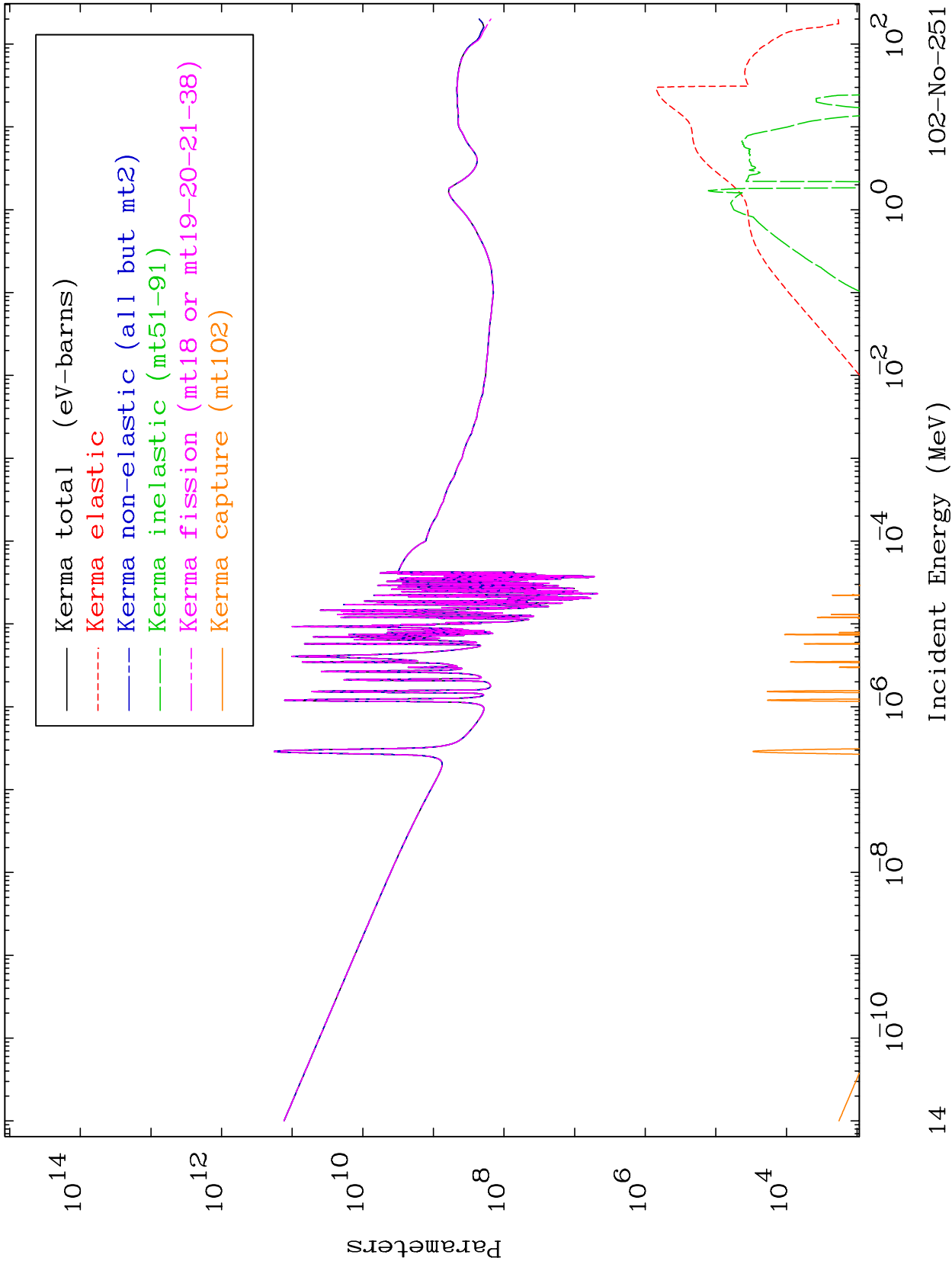


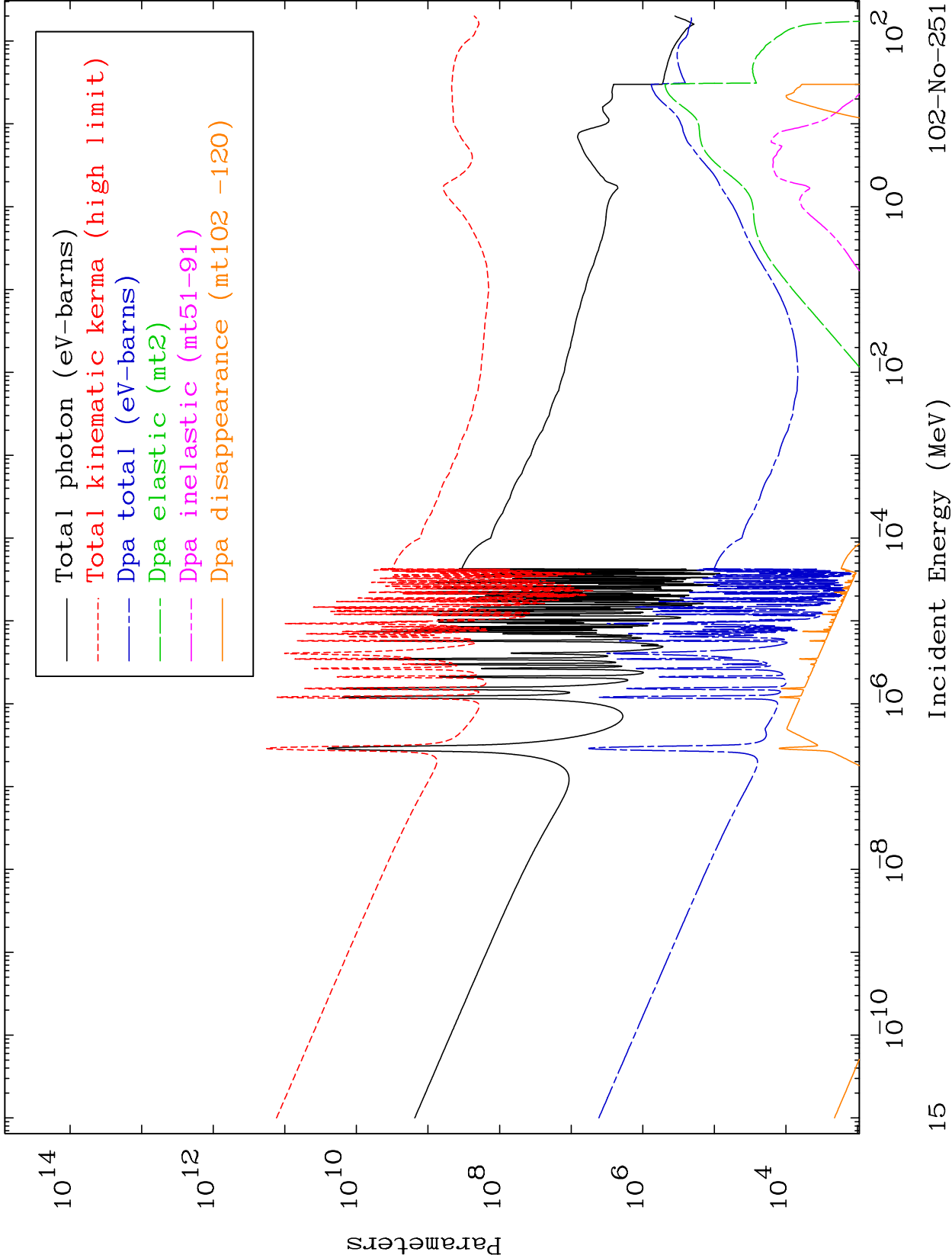
12

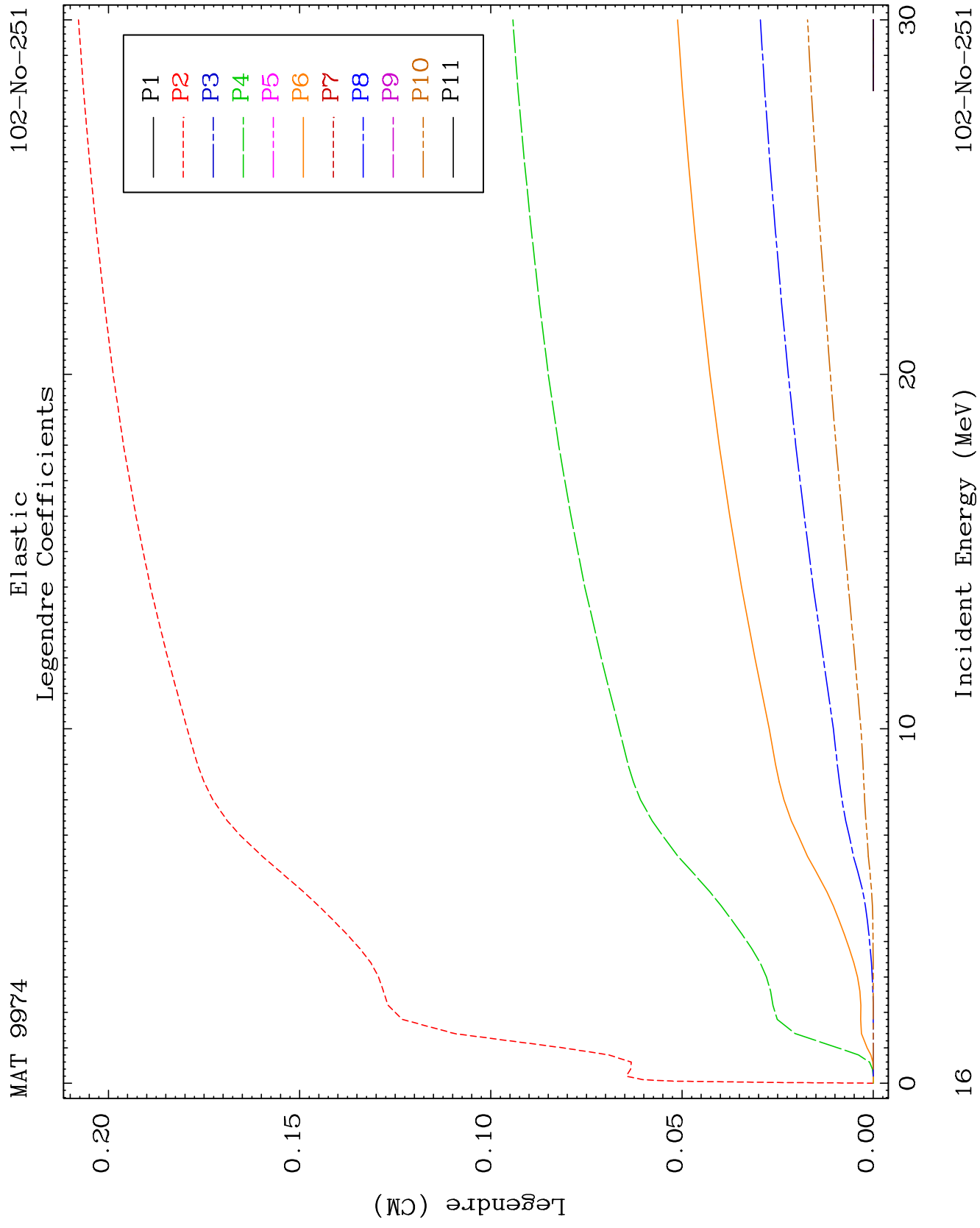
Incident Energy (MeV)

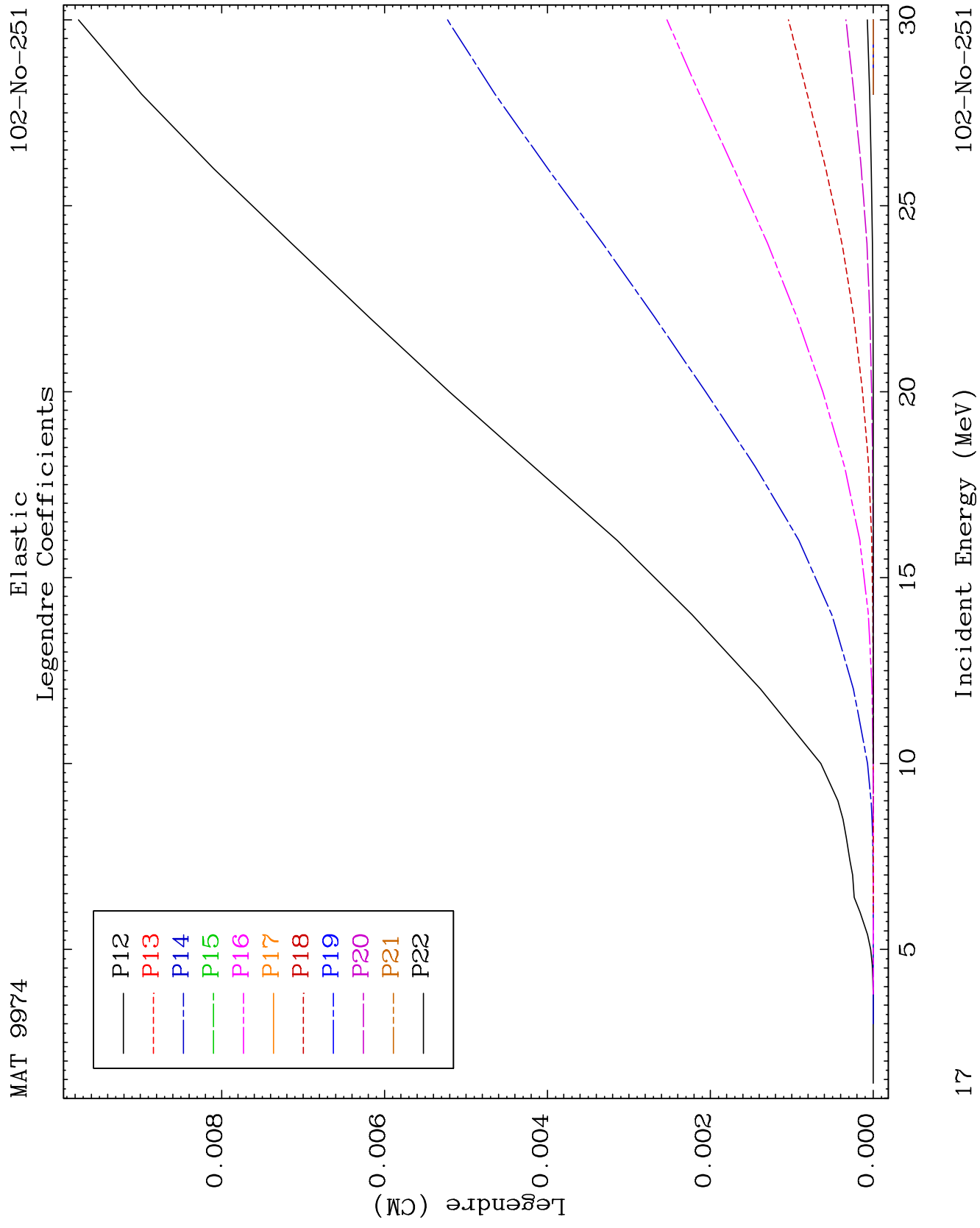
102-No-251

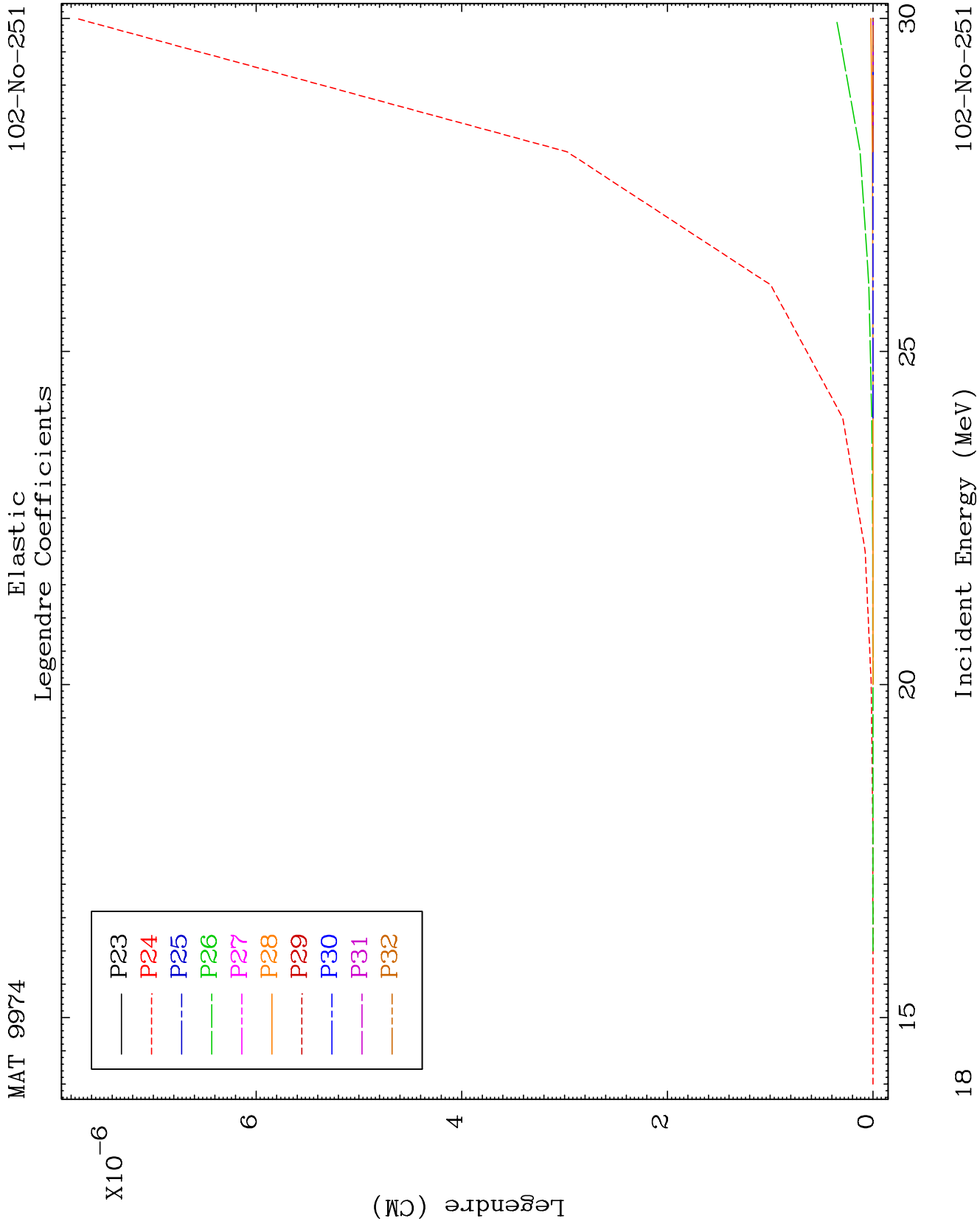








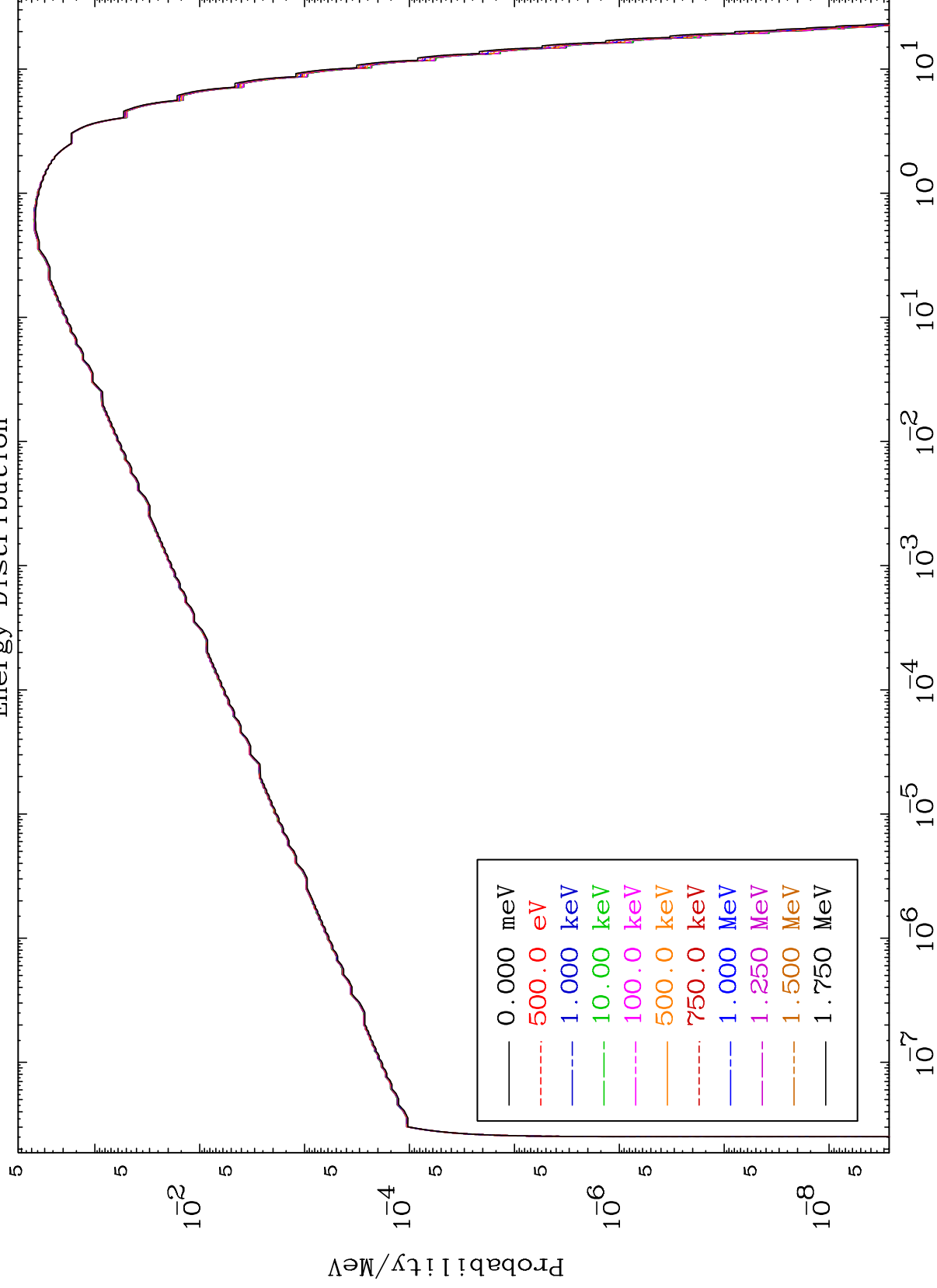


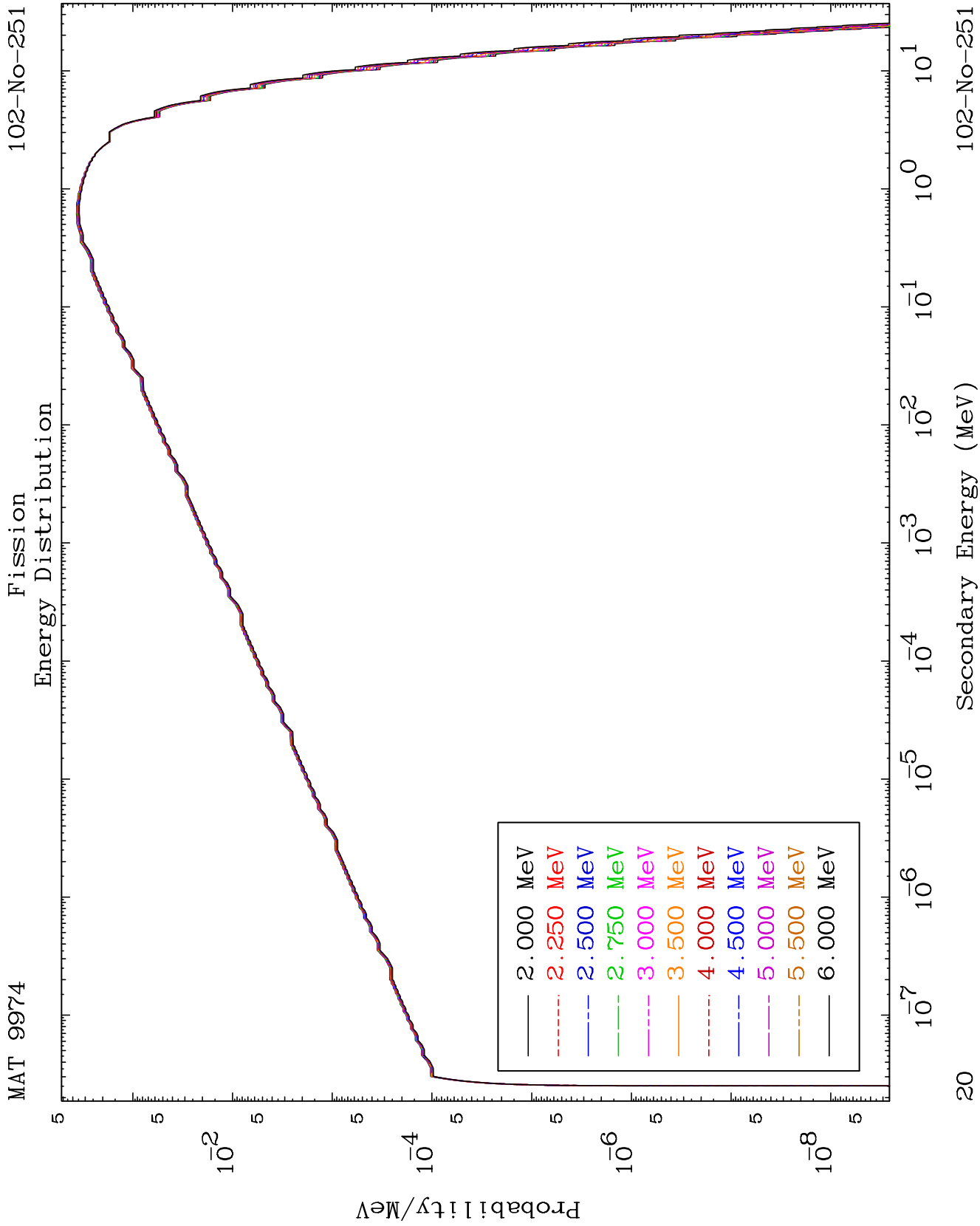


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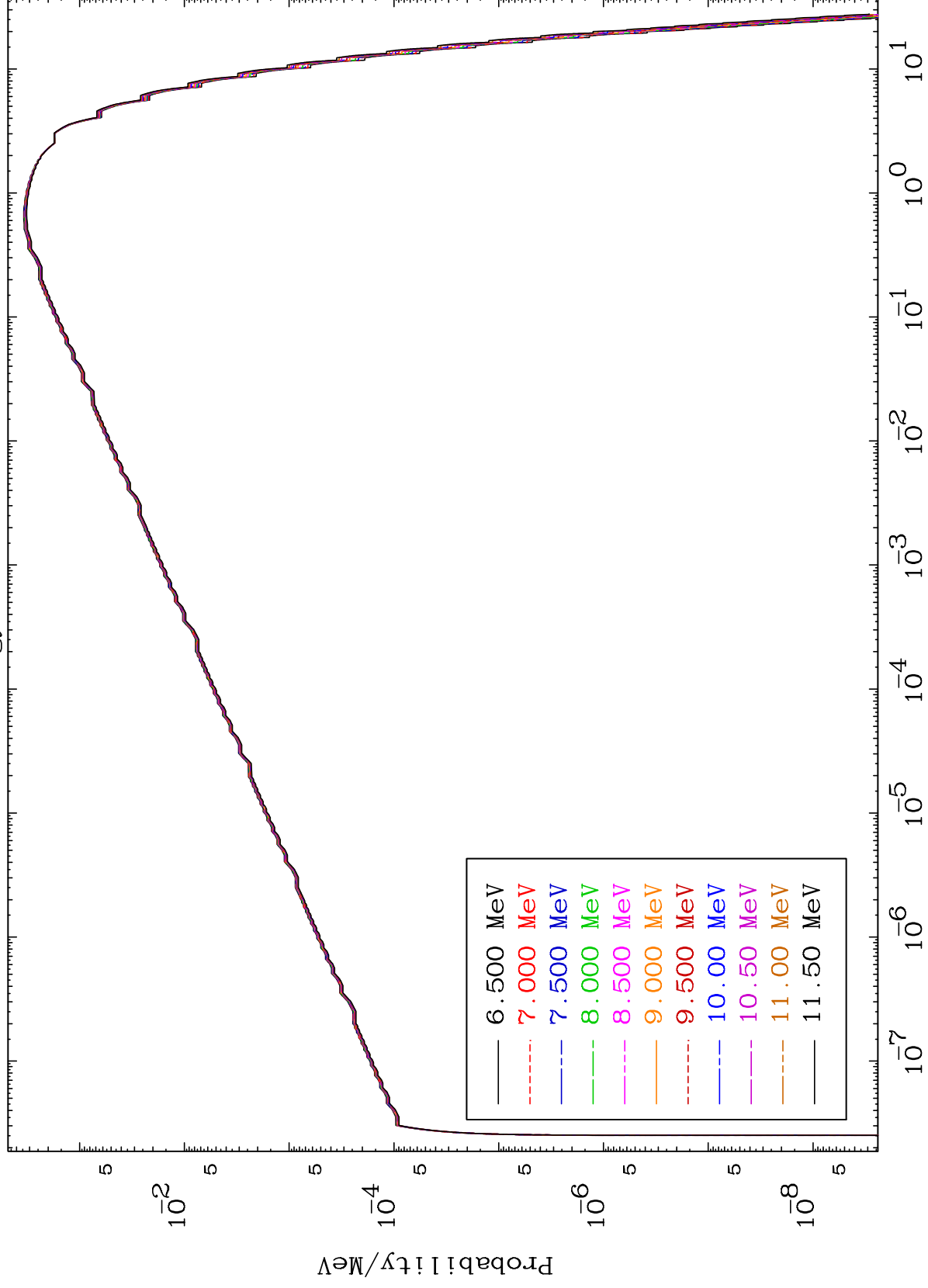
Fission
Energy Distribution

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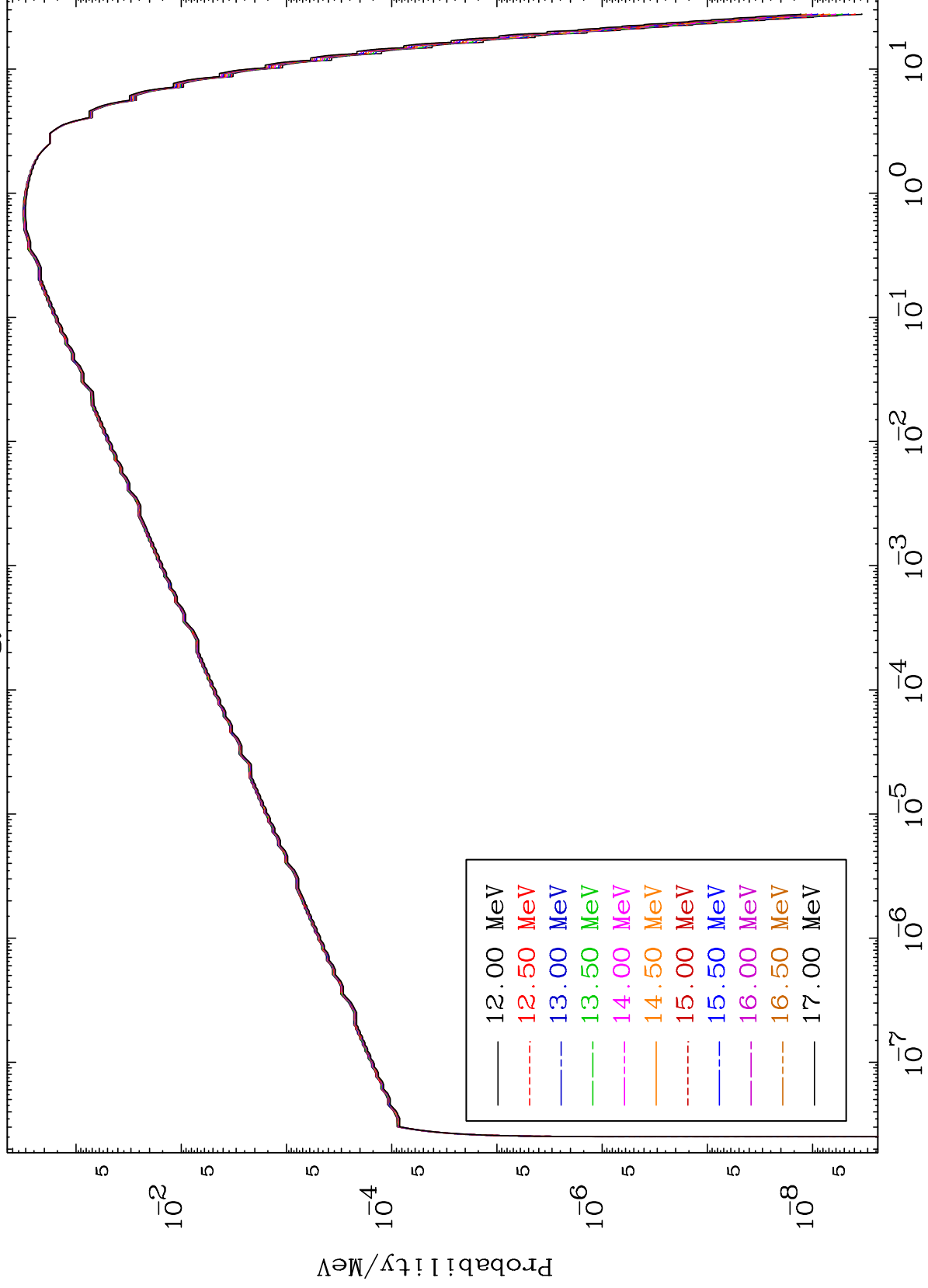




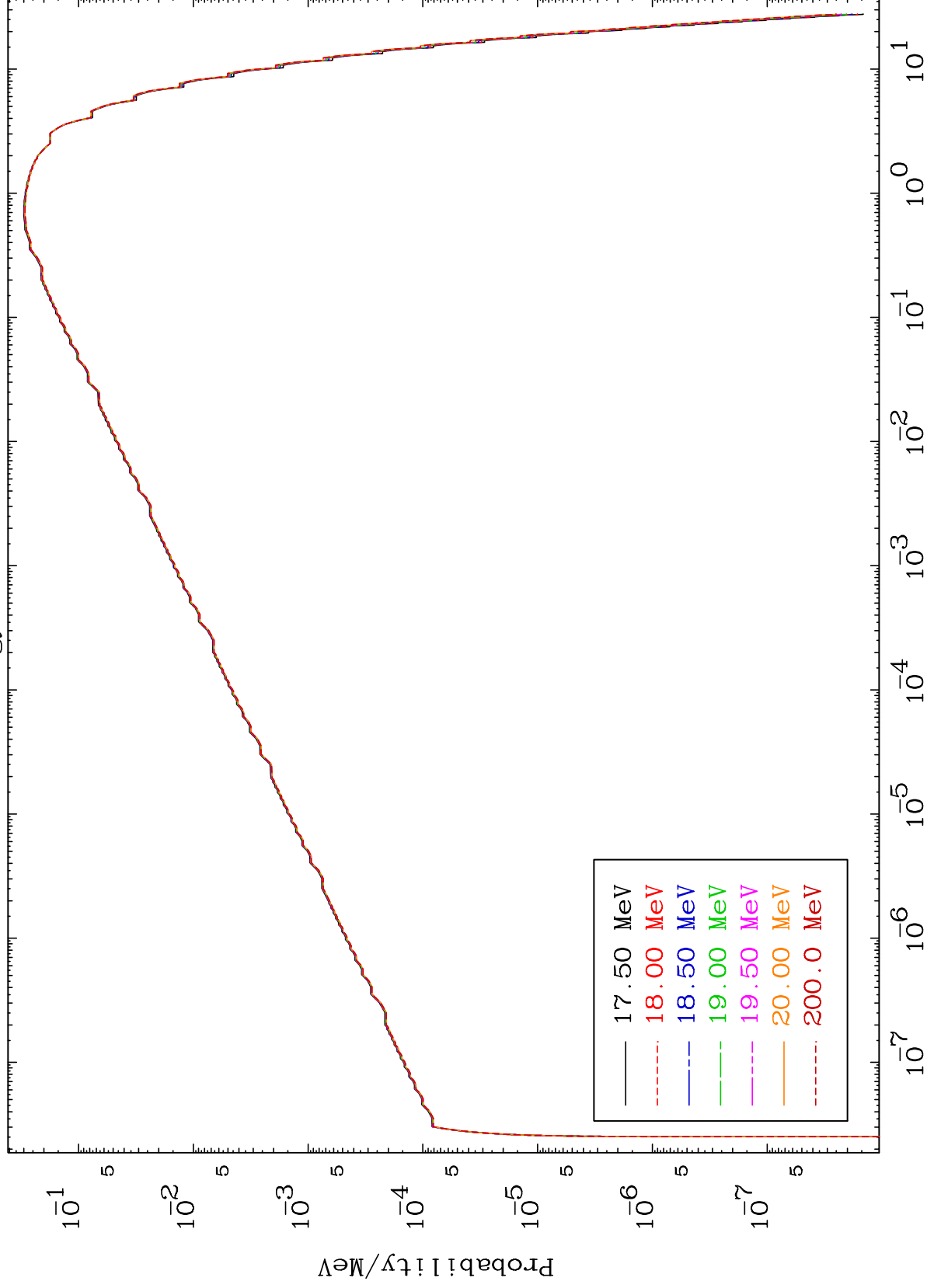
Energy Distribution



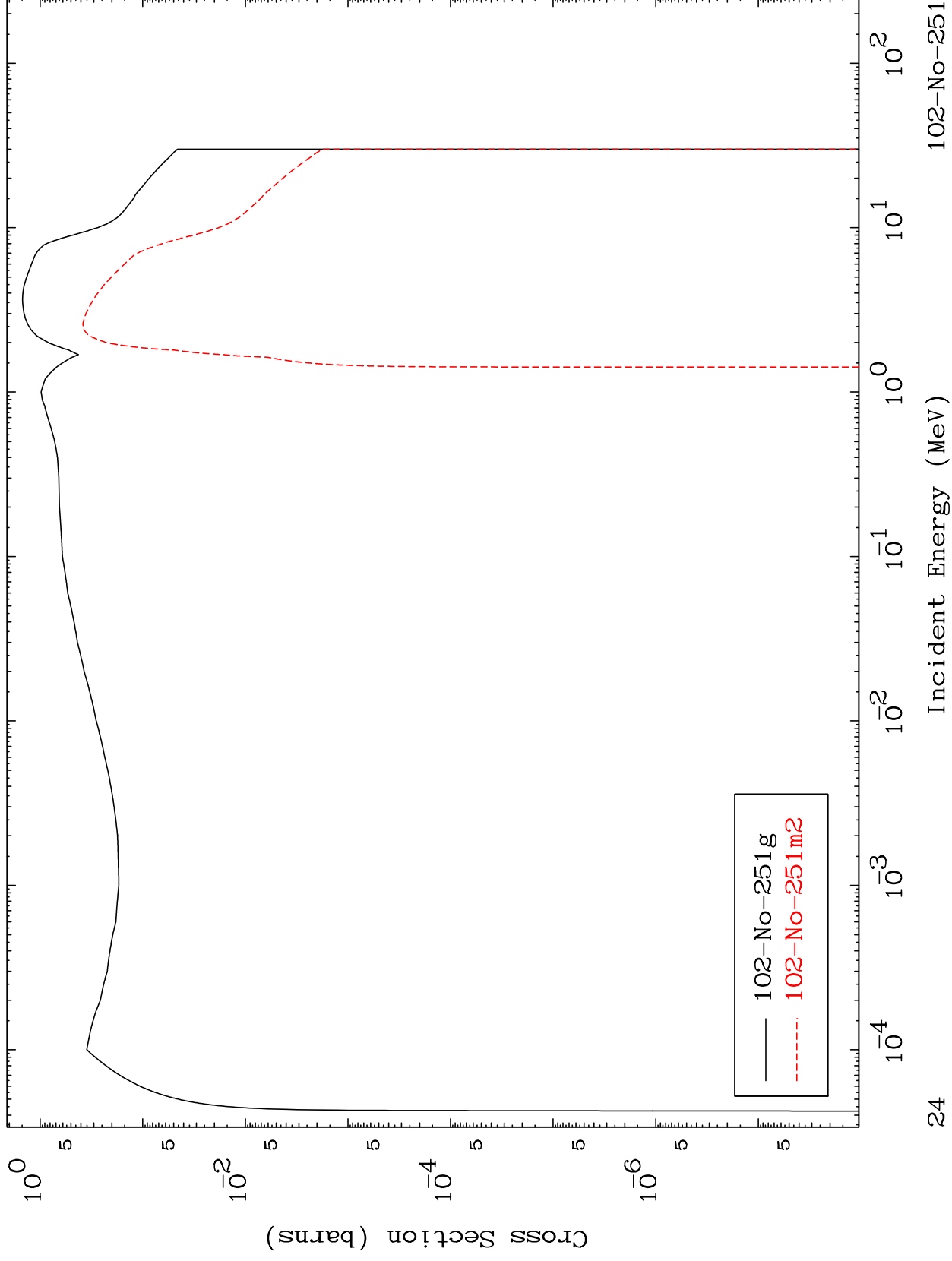
Energy Distribution



Energy Distribution



Radionuclide Production Cross Section

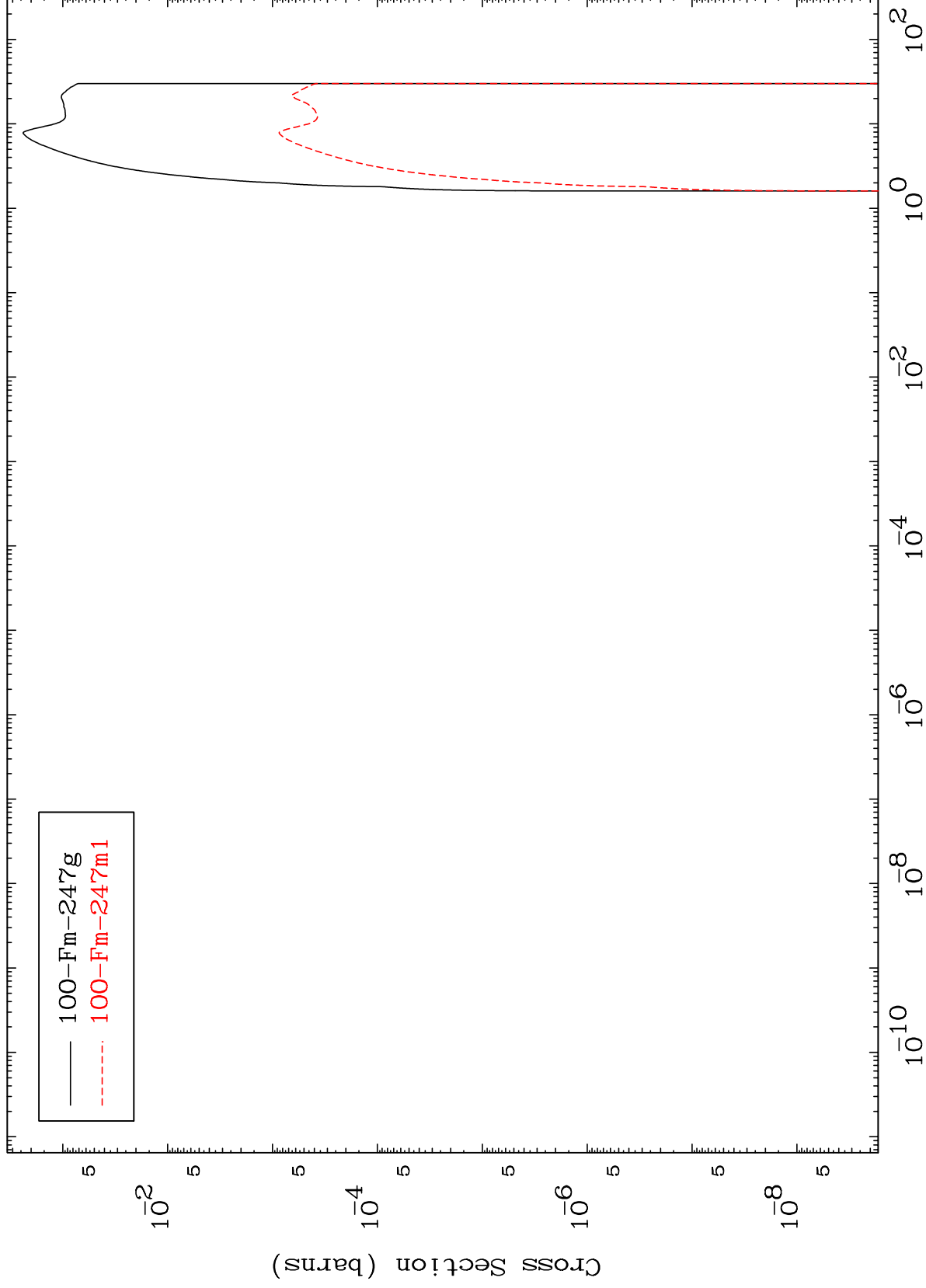


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$(n,n') \alpha$

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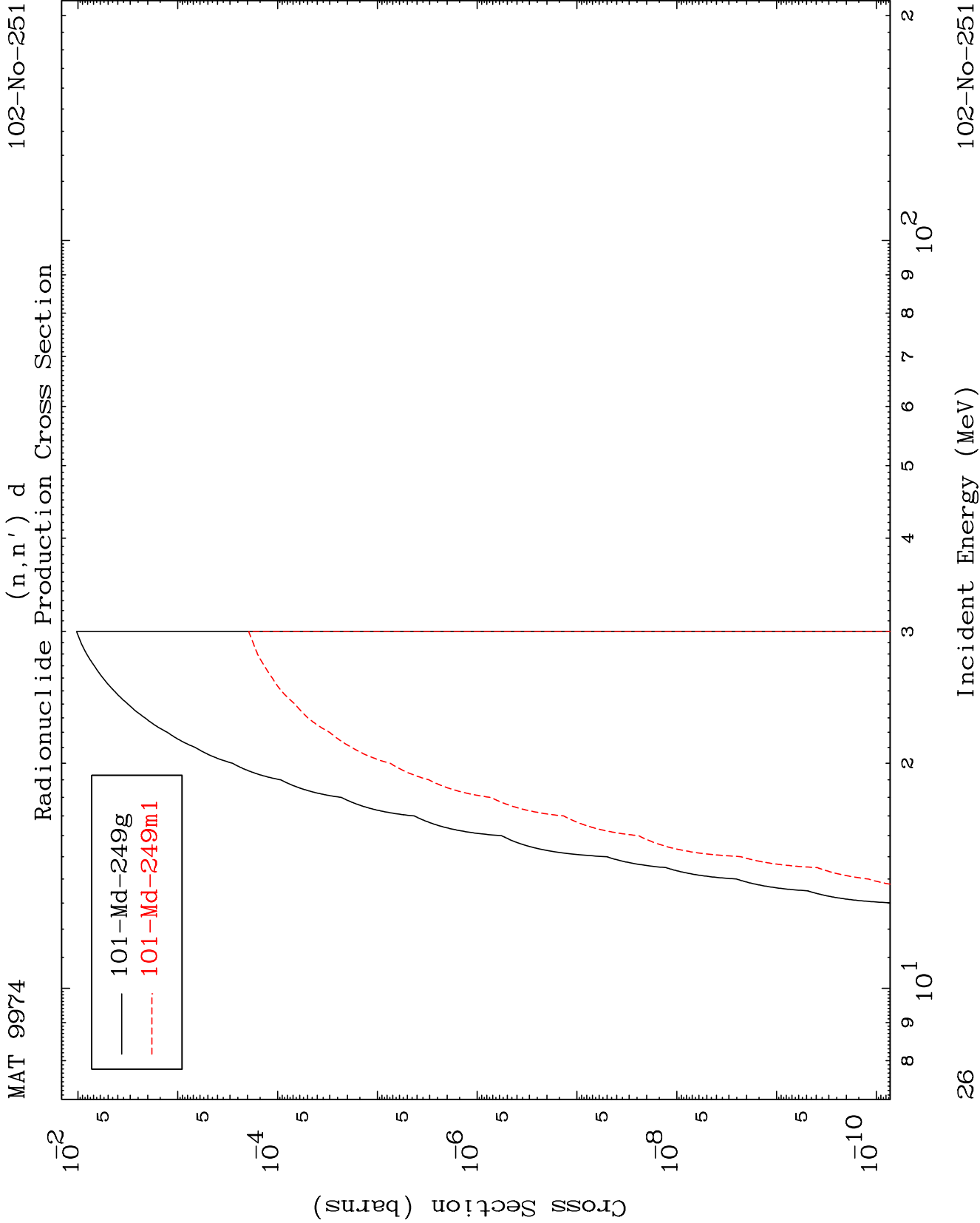
Radionuclide Production Cross Section



25

Incident Energy (MeV)

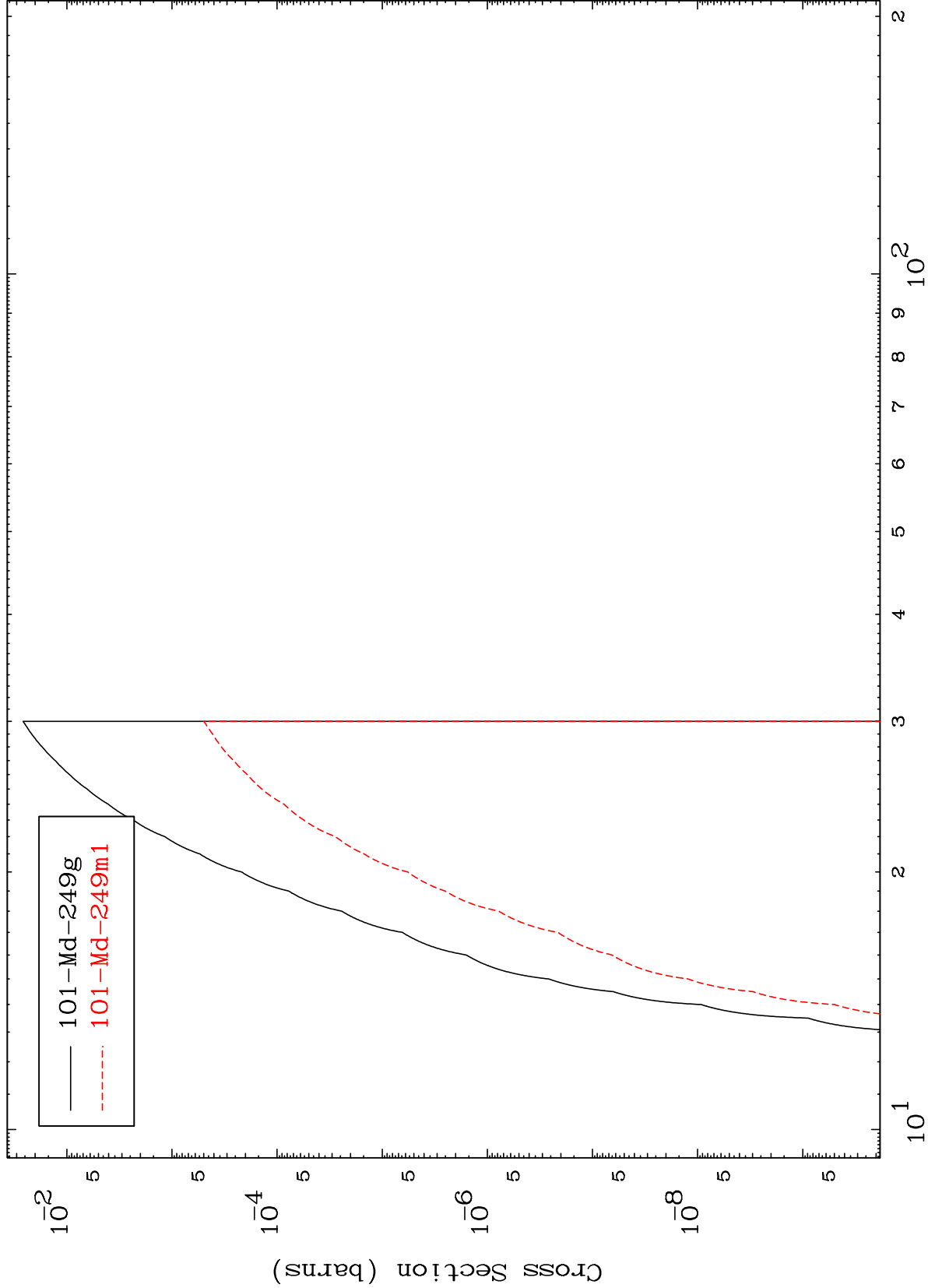
102-No-251



MAT 9974

(n,2n) p 102-No-251

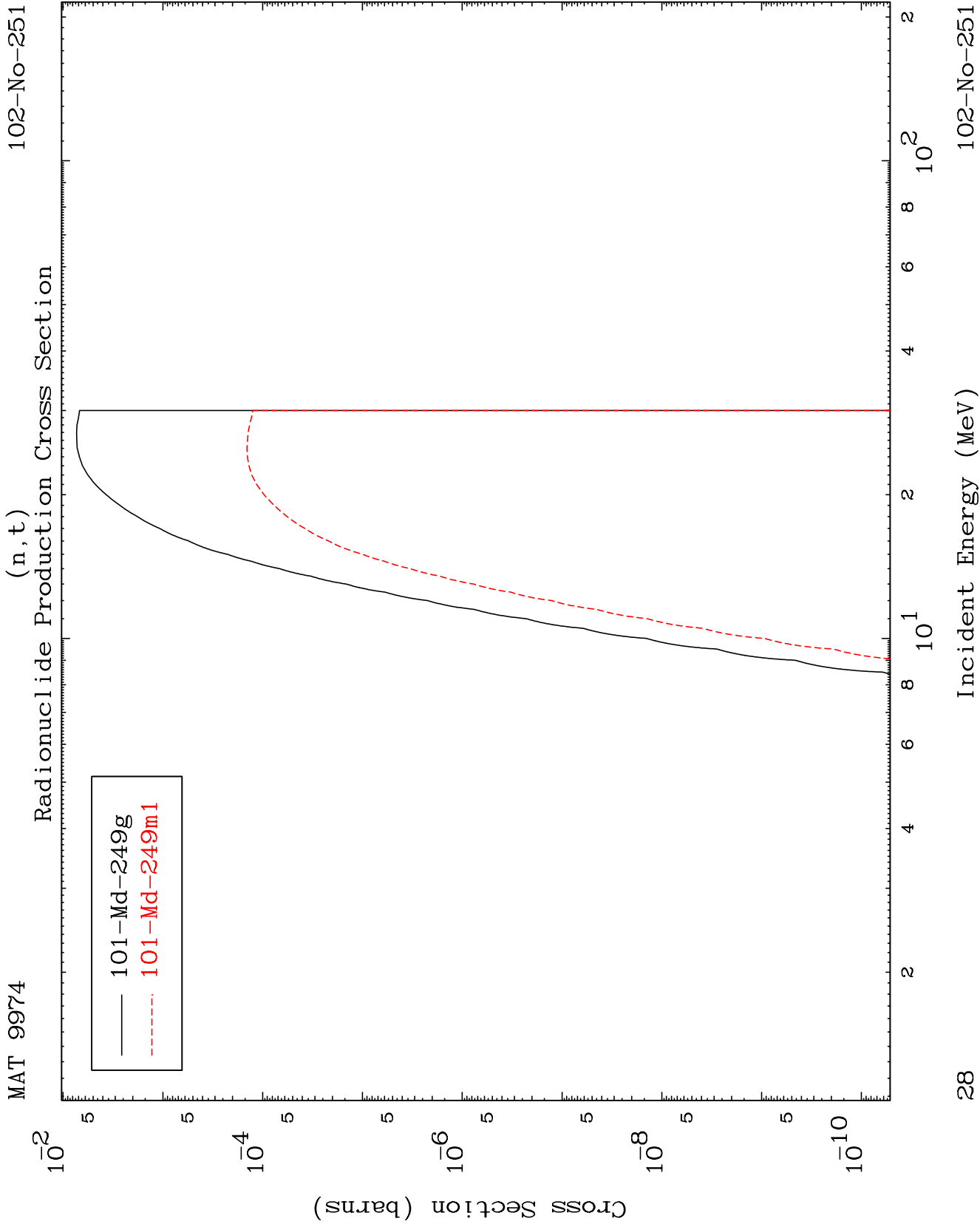
Radionuclide Production Cross Section



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Incident Energy (MeV)

27

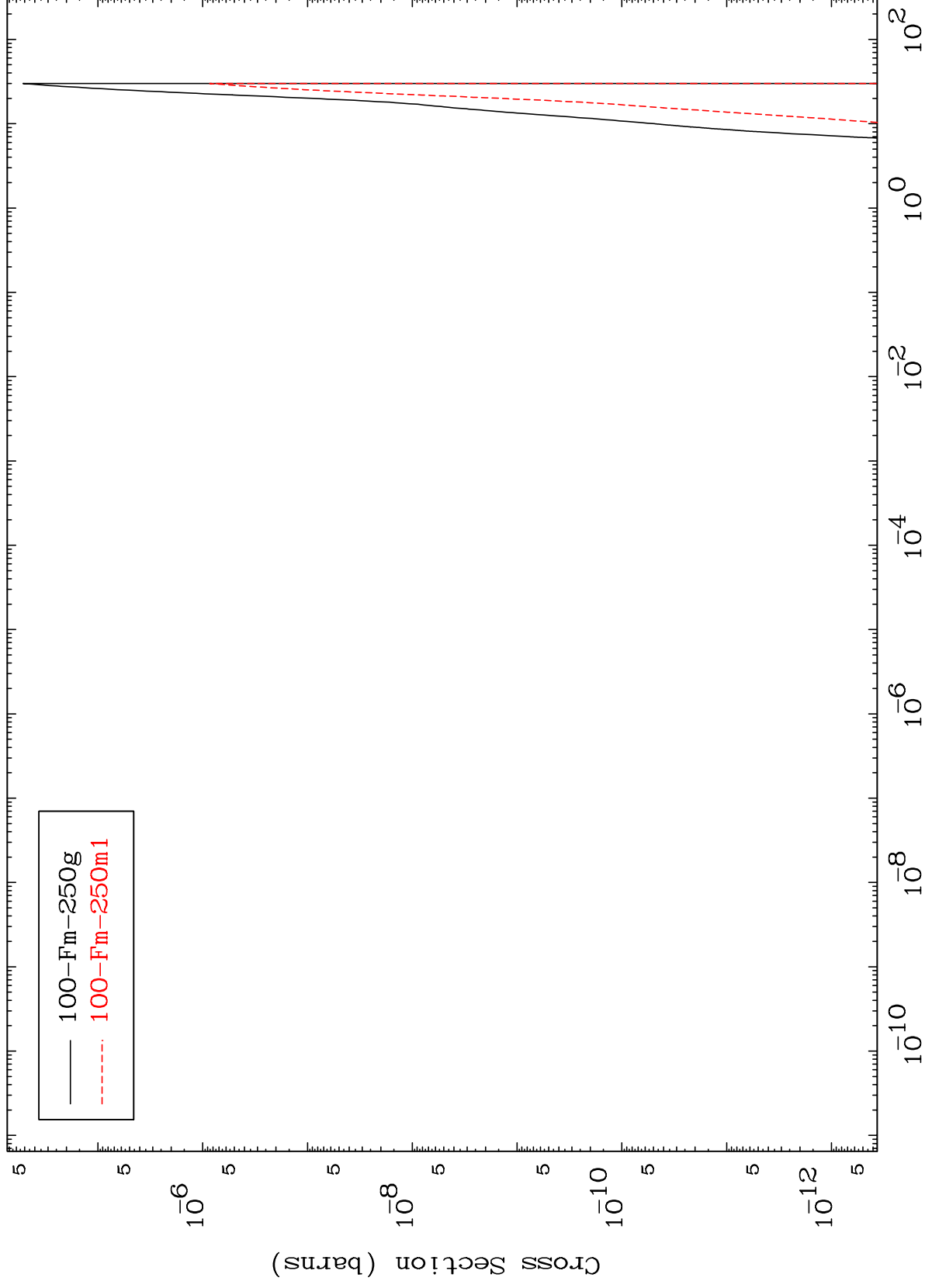


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(n,2p)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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