

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

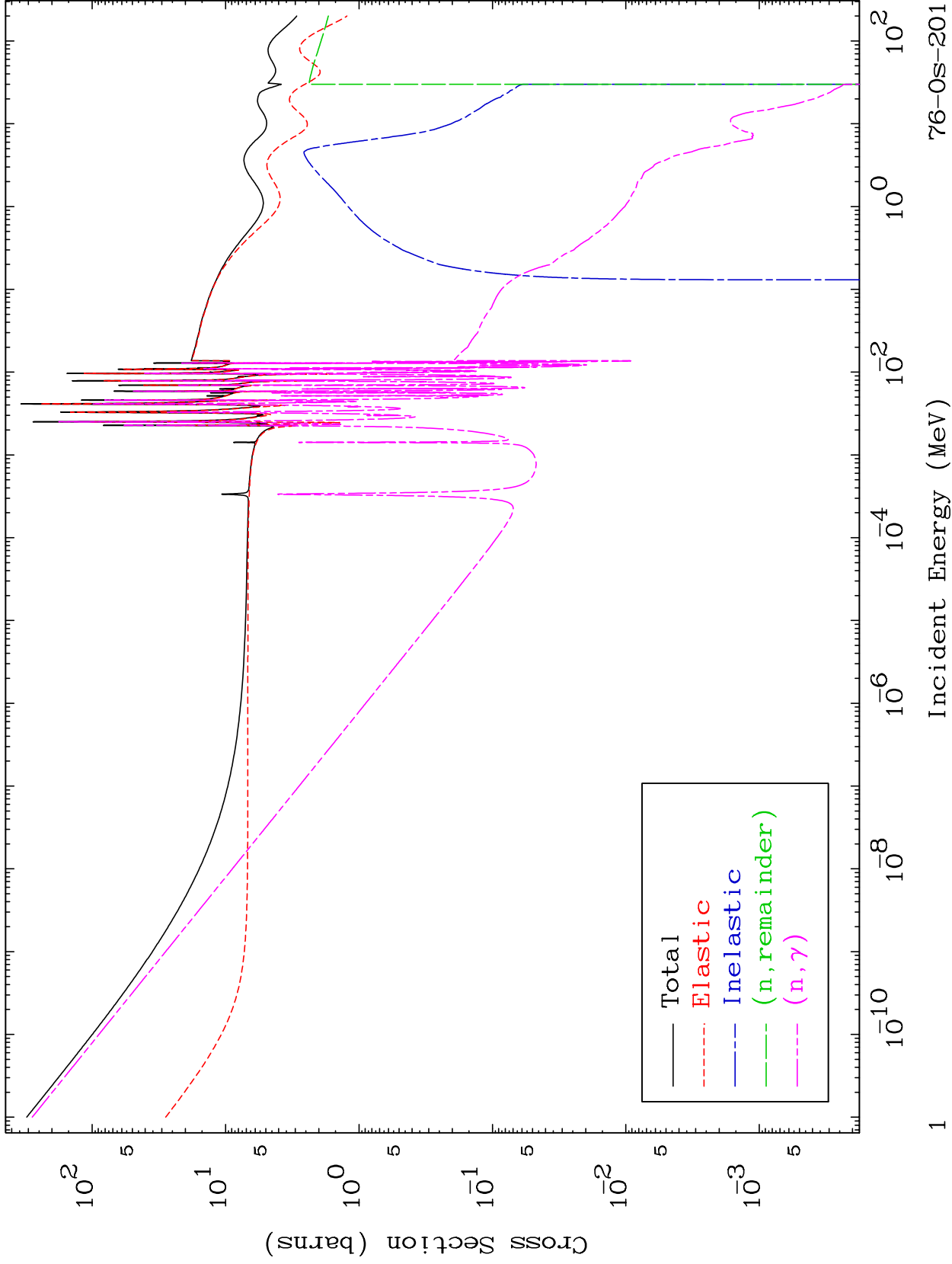
Press Mouse Button to Start

MAT 7676

Major

293 Kelvin Cross Sections

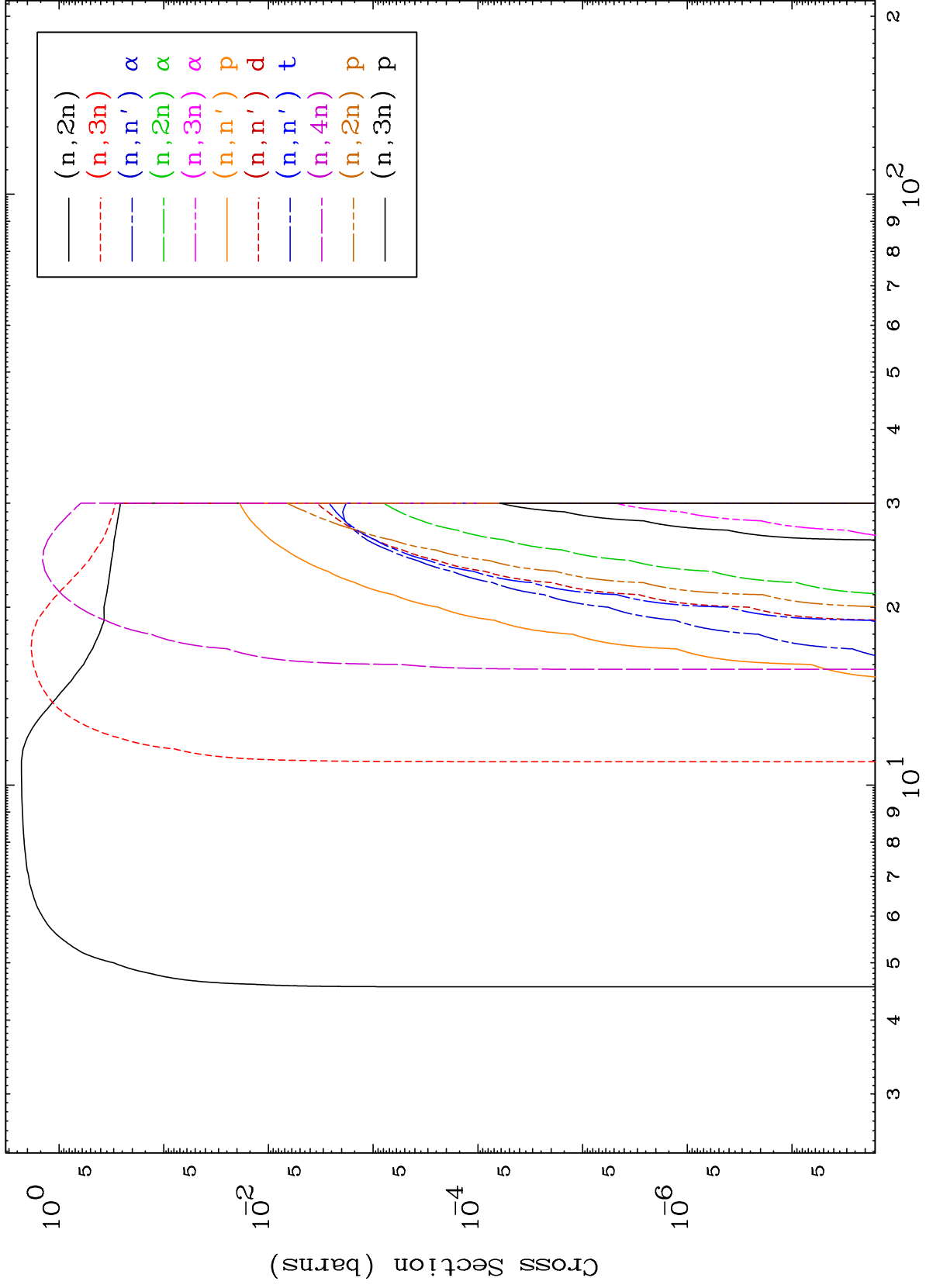
76-Os-201



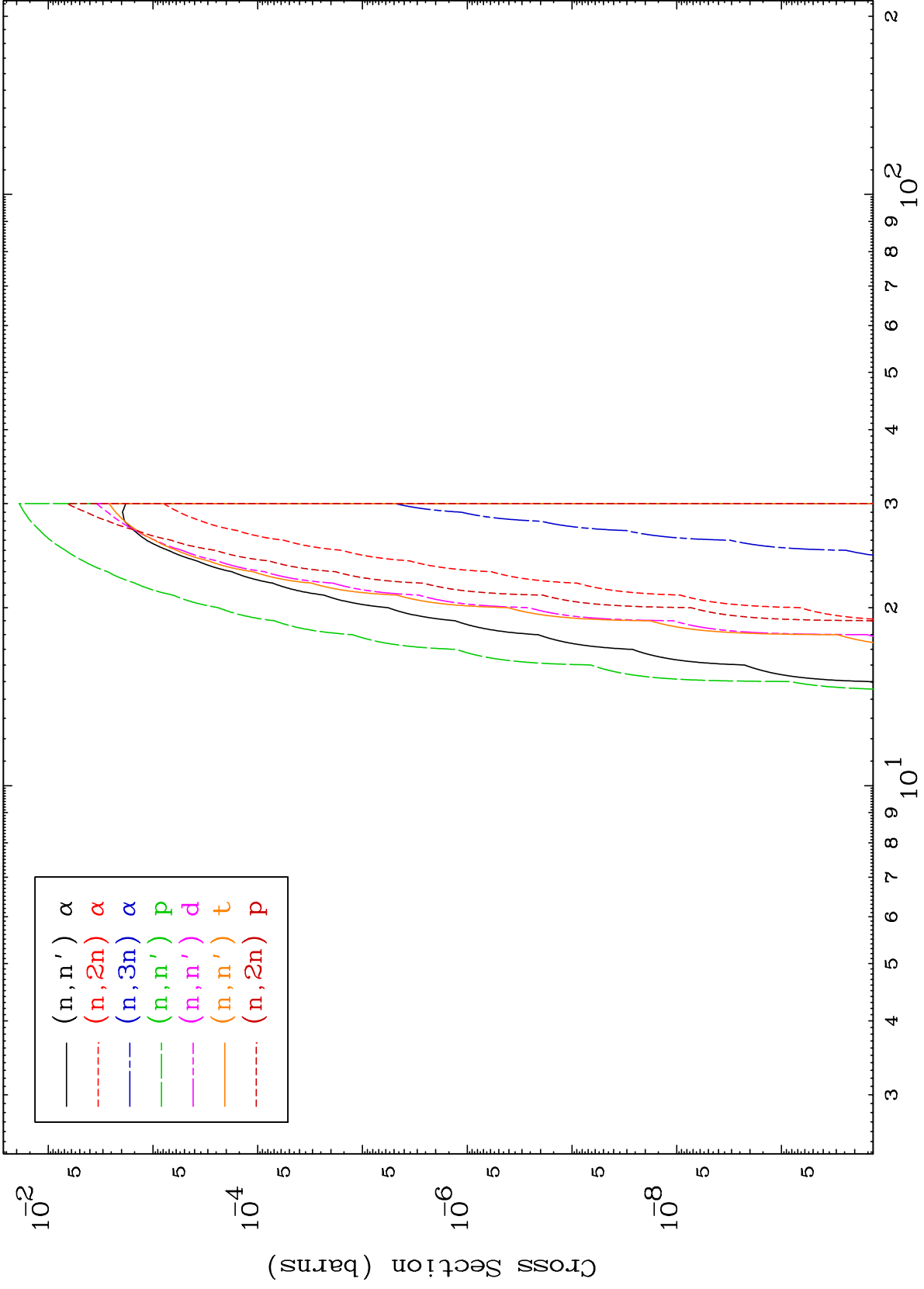
MAT 7676

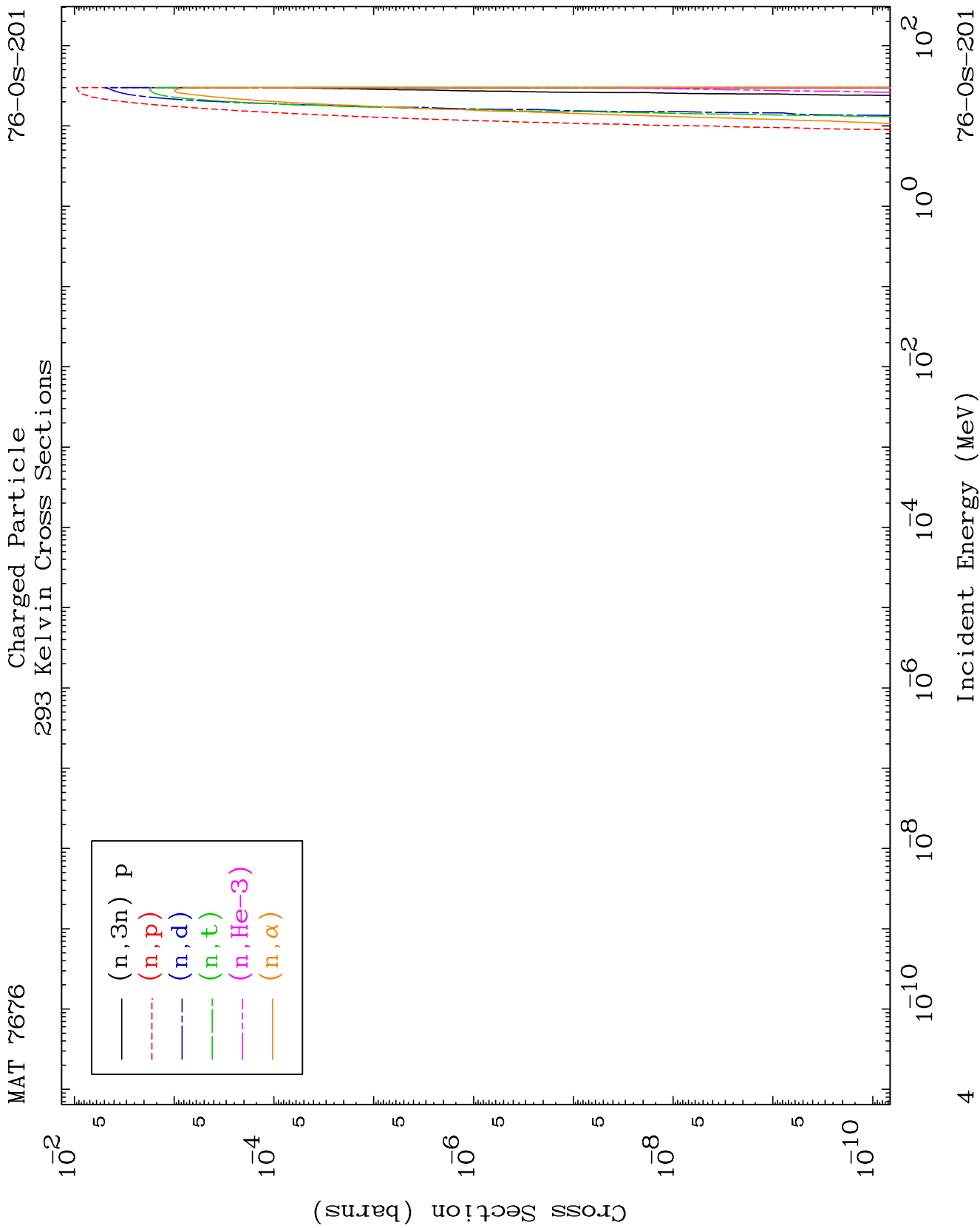
Neutron Production
293 Kelvin Cross Sections

76-0s-201



76-0s-201

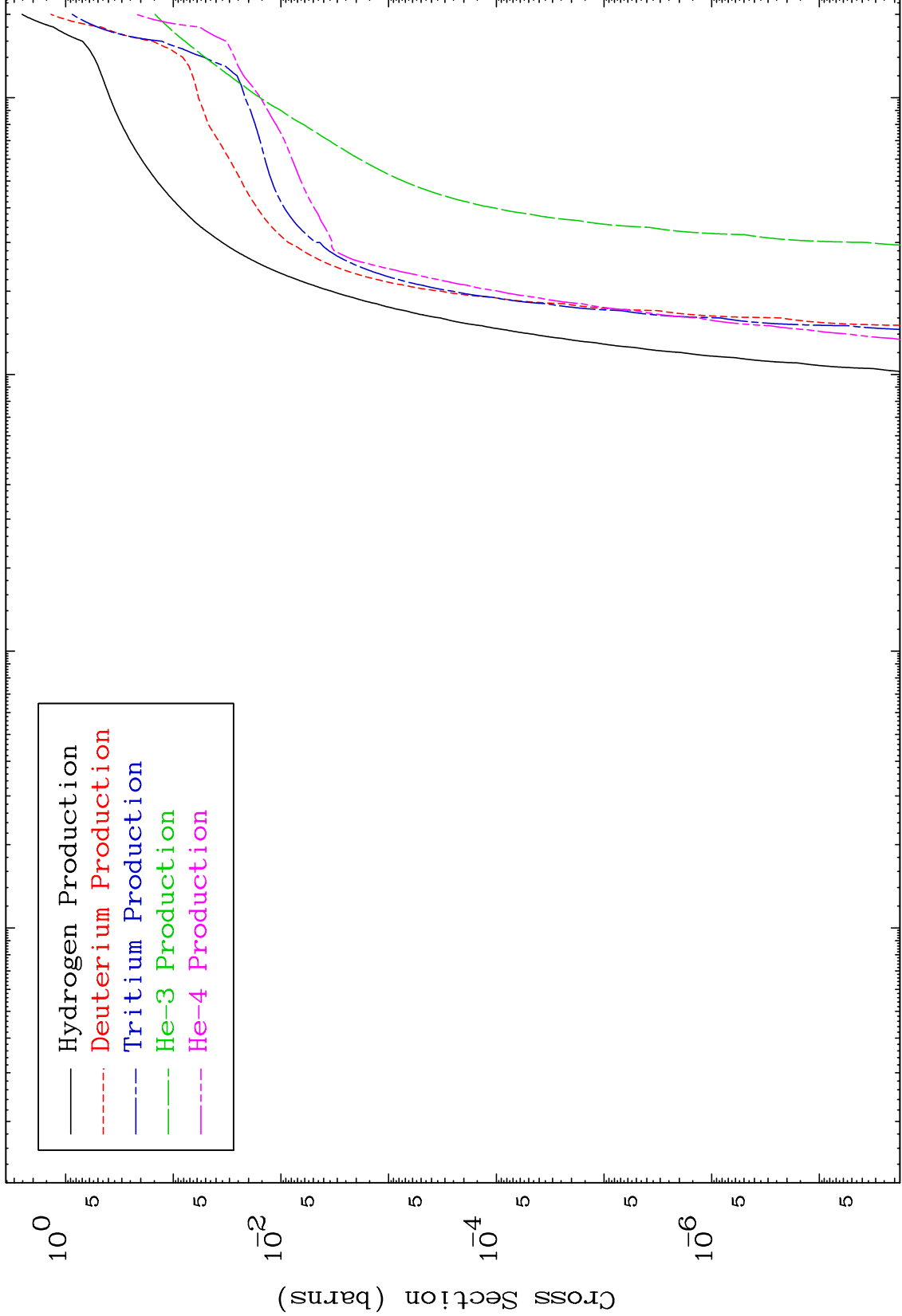




MAT 7676

Particle Production
293 Kelvin Cross Sections

76-0s-201



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Incident Energy (MeV)

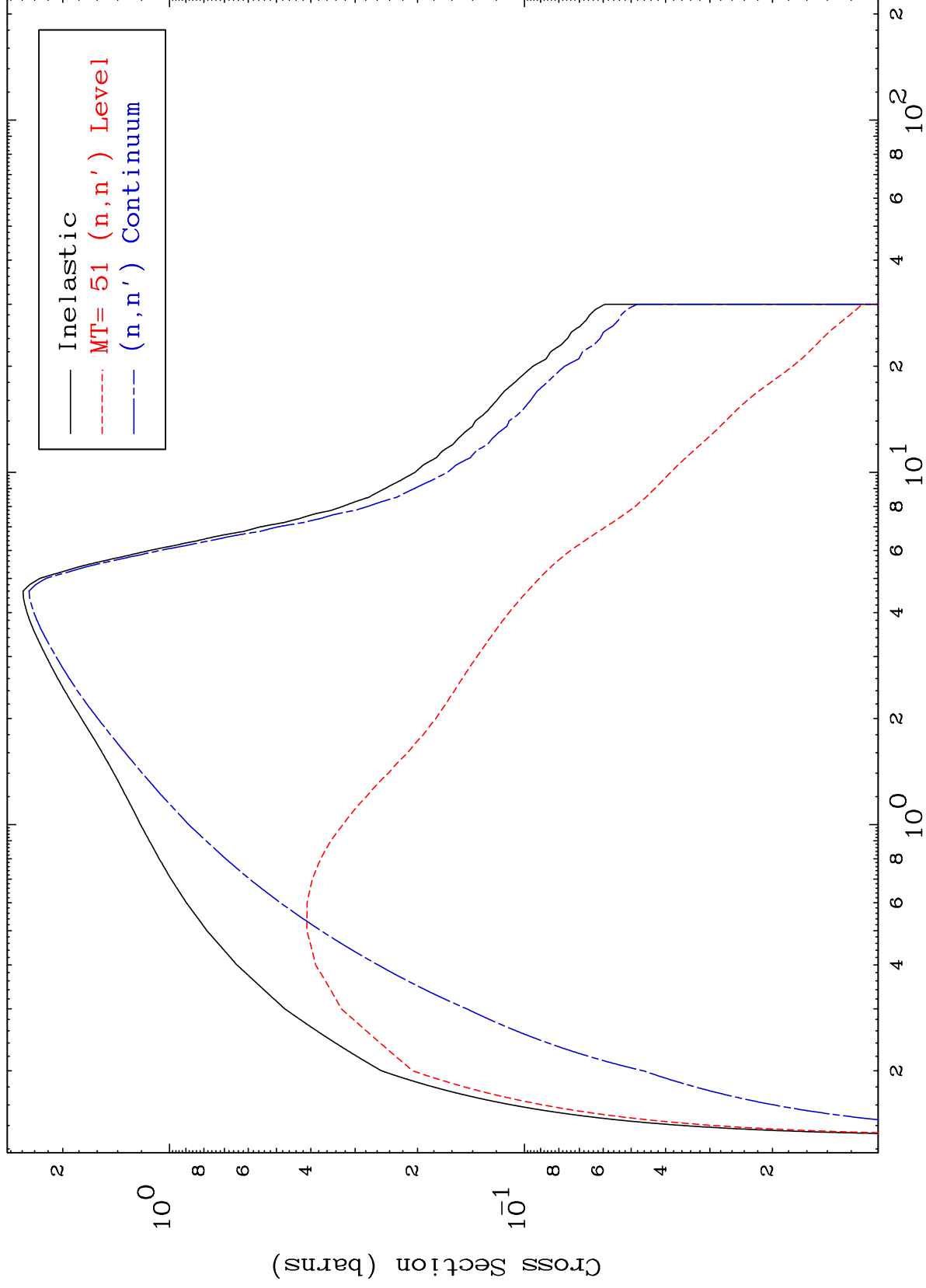
76-0s-201

MAT 7676

(n,n') Level

76-Os-201

293 Kelvin Cross Sections



6

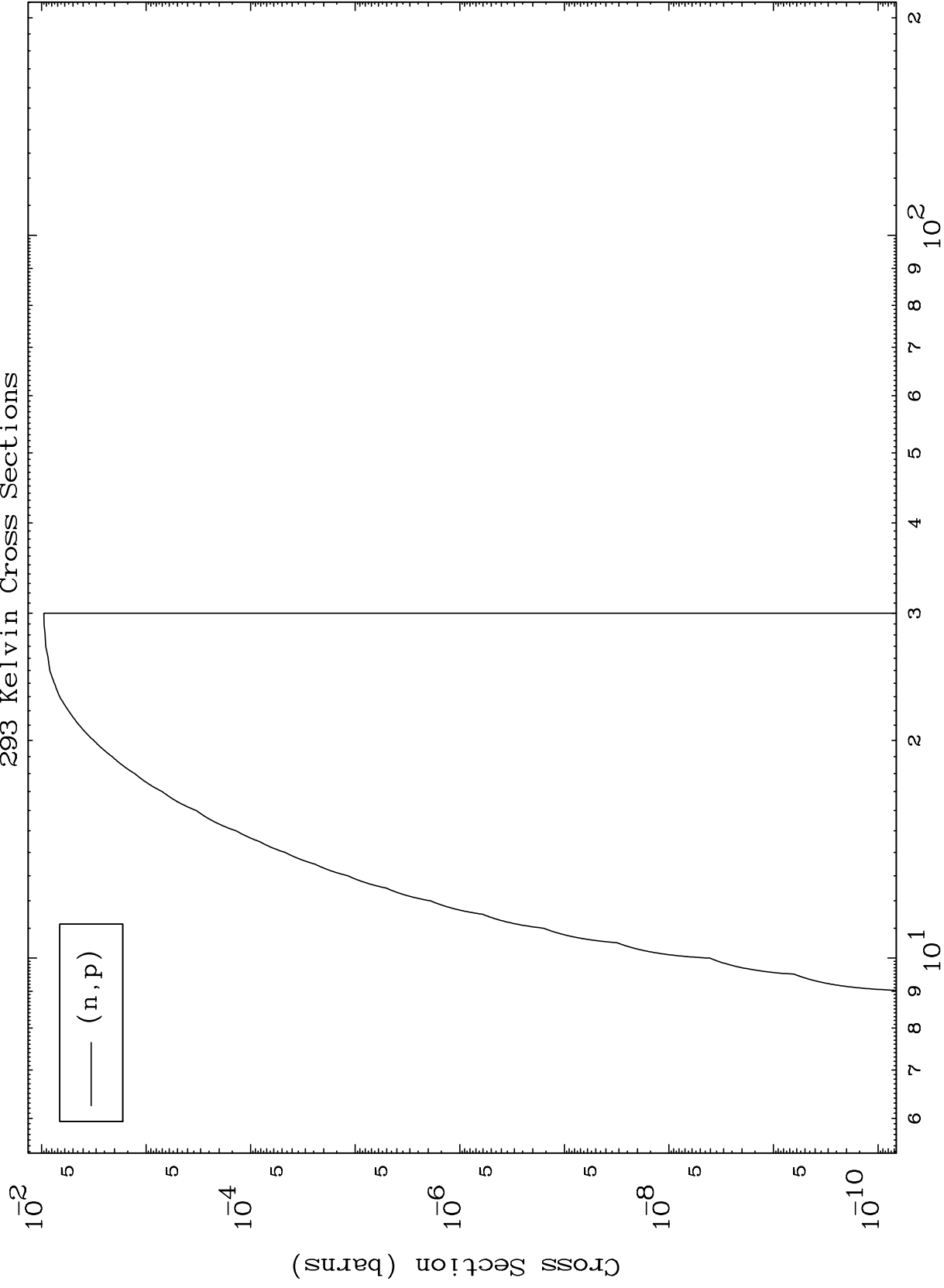
Incident Energy (MeV)

76-Os-201

MAT 7676

(n,p) Levels
293 Kelvin Cross Sections

76-0s-201



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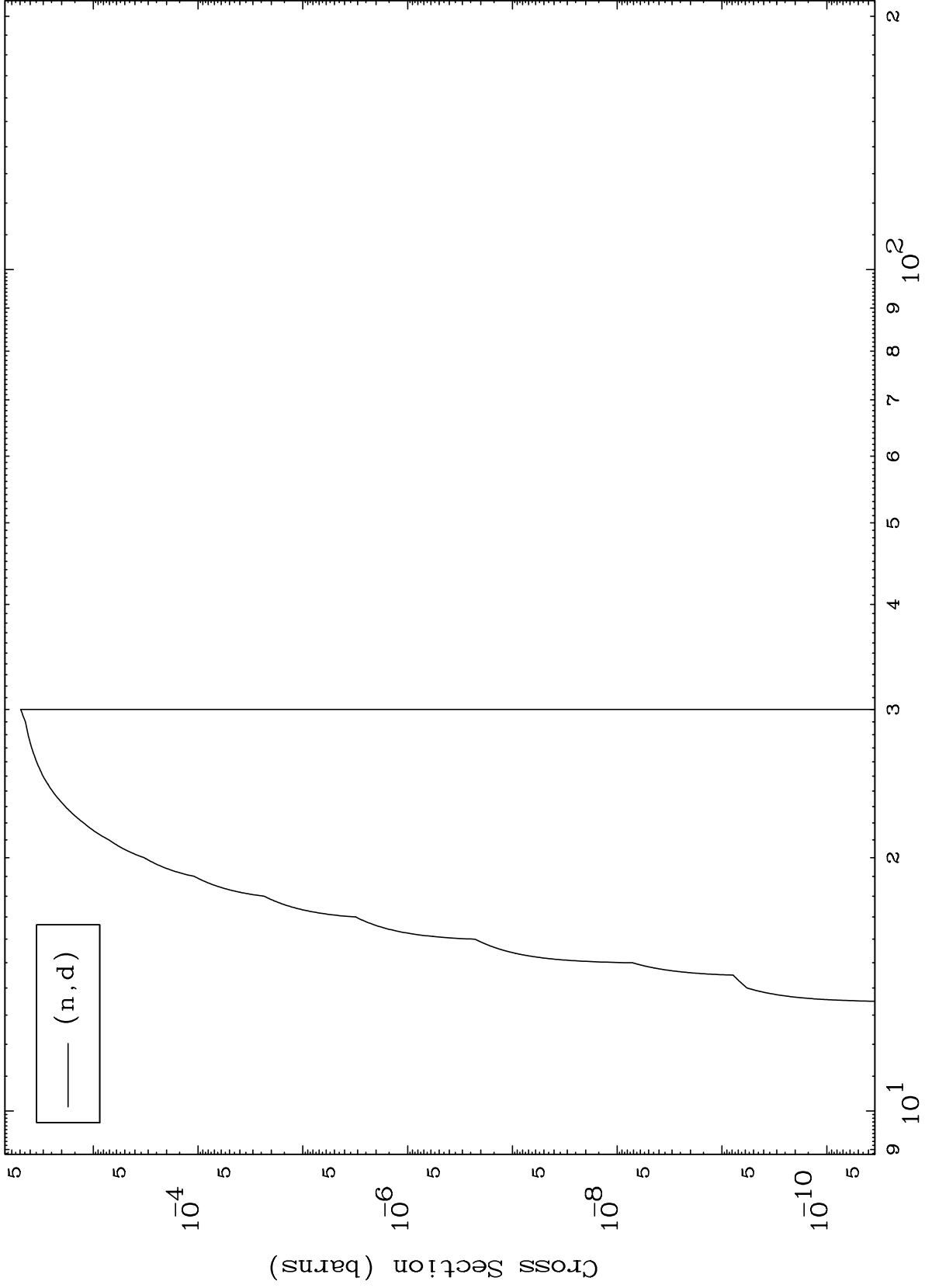
Incident Energy (MeV)

76-0s-201

MAT 7676

(n,d) Levels
293 Kelvin Cross Sections

76-0s-201



8

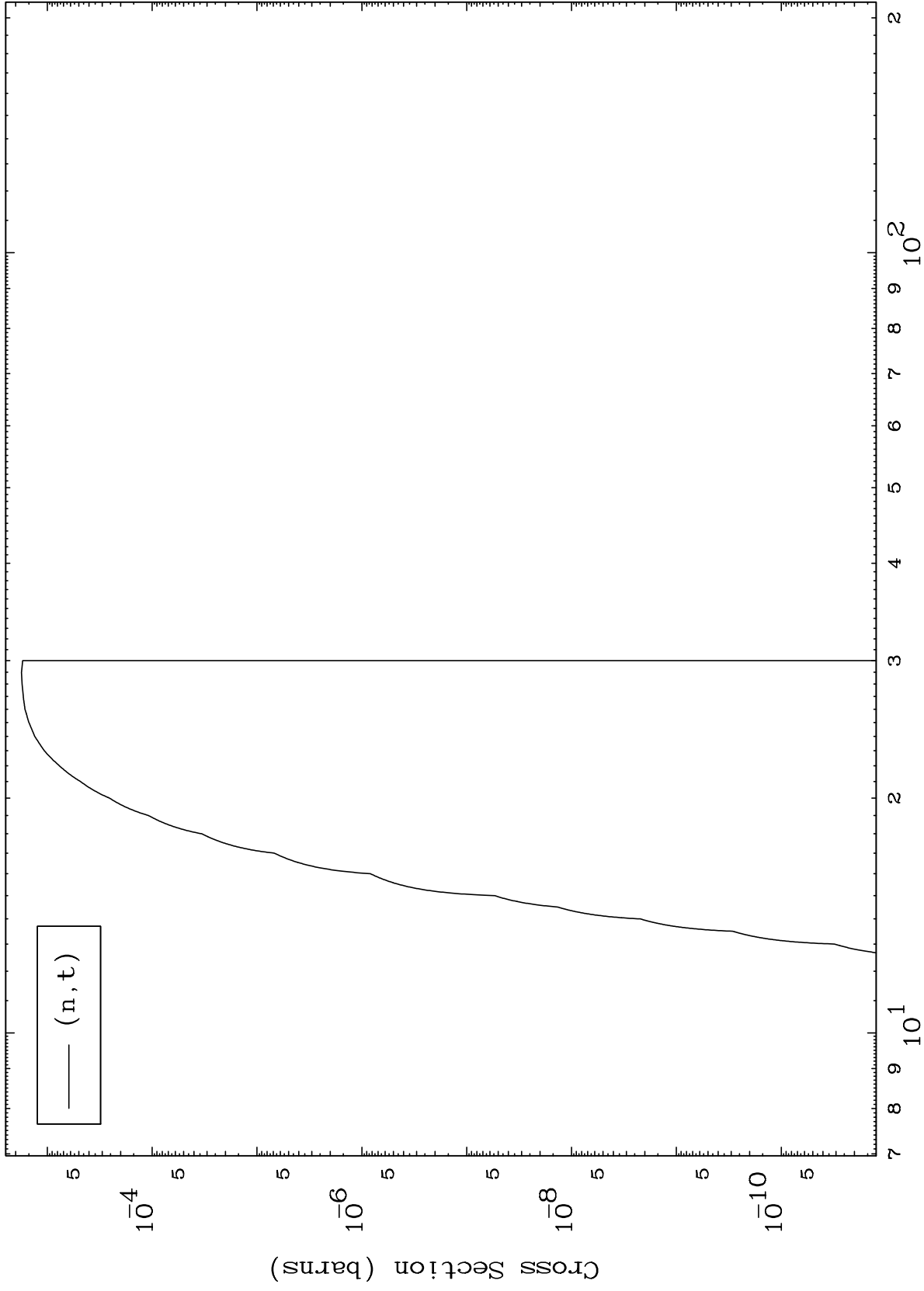
Incident Energy (MeV)

76-0s-201

MAT 7676

(n,t) Levels
293 Kelvin Cross Sections

76-0s-201



9

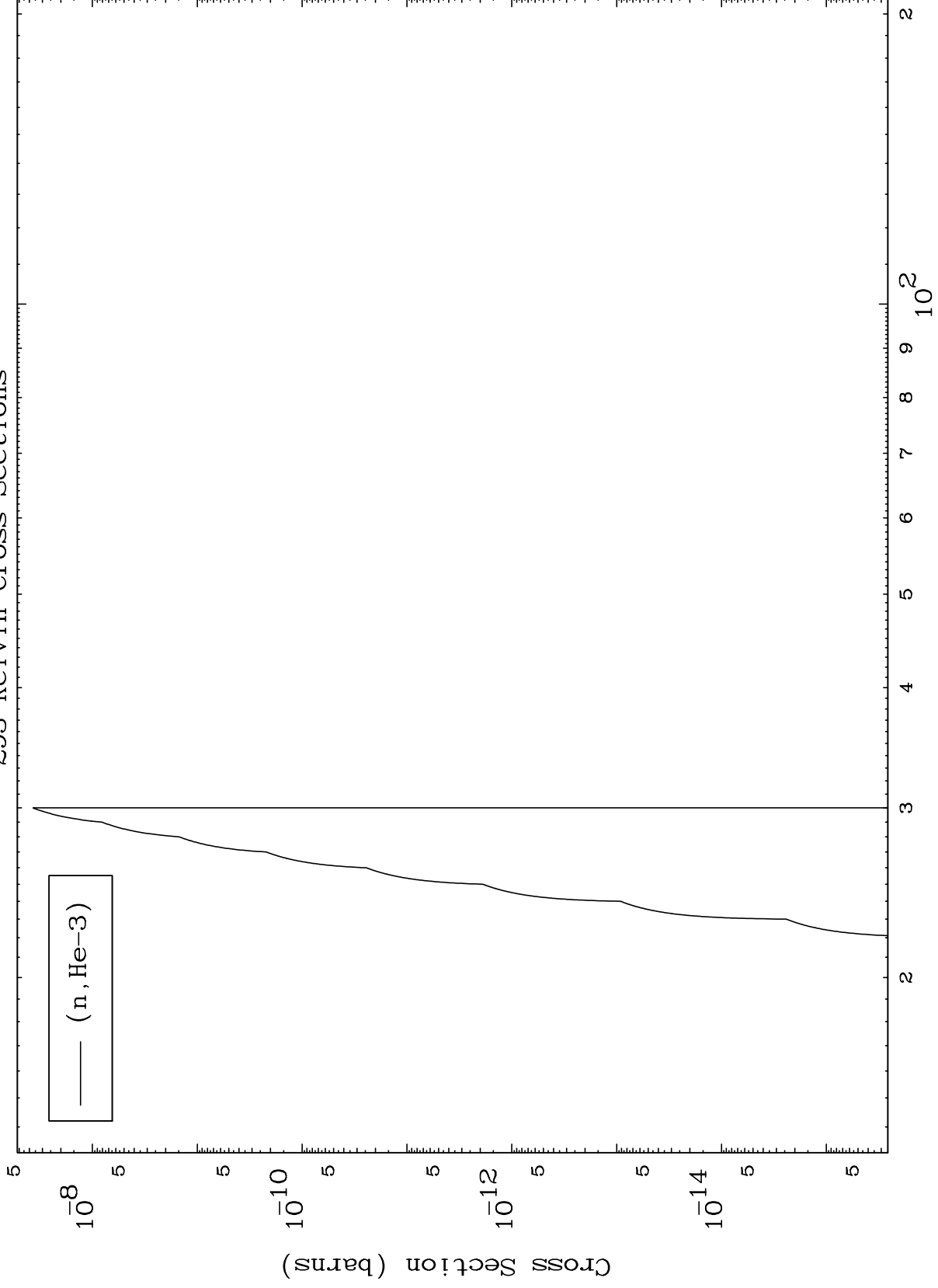
Incident Energy (MeV)

76-0s-201

MAT 7676

(n,He3) Levels
293 Kelvin Cross Sections

76-0s-201



10

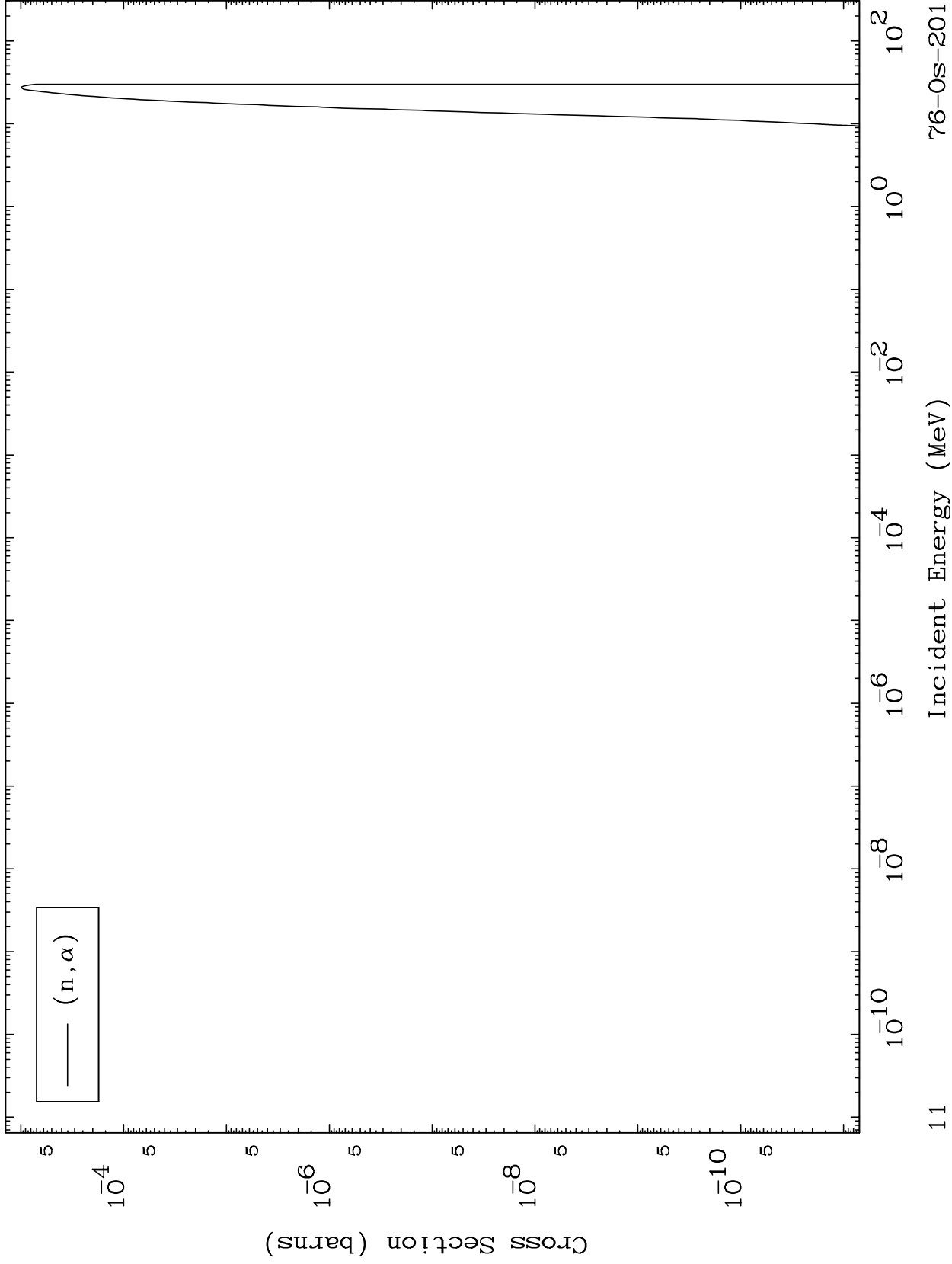
Incident Energy (MeV)

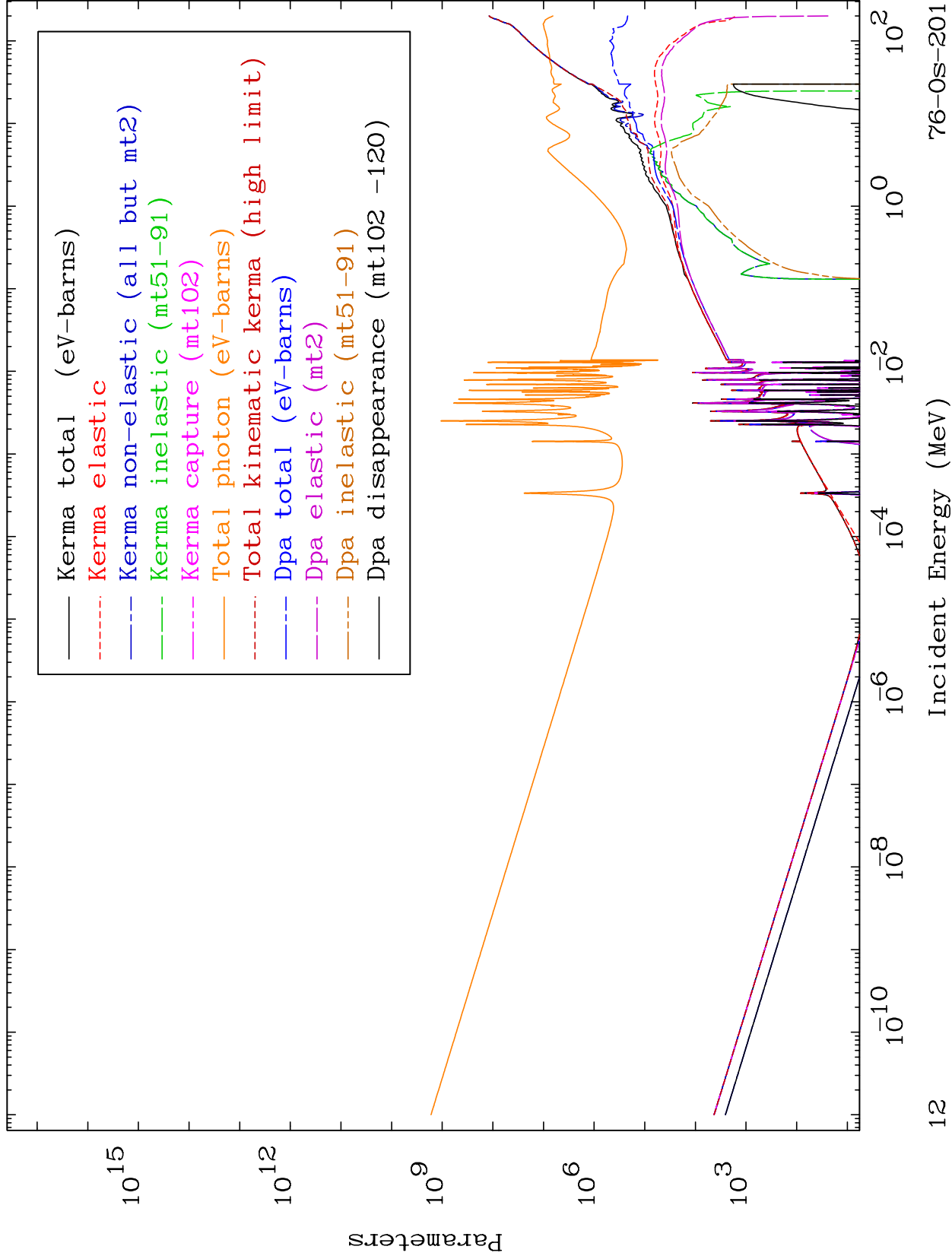
76-0s-201

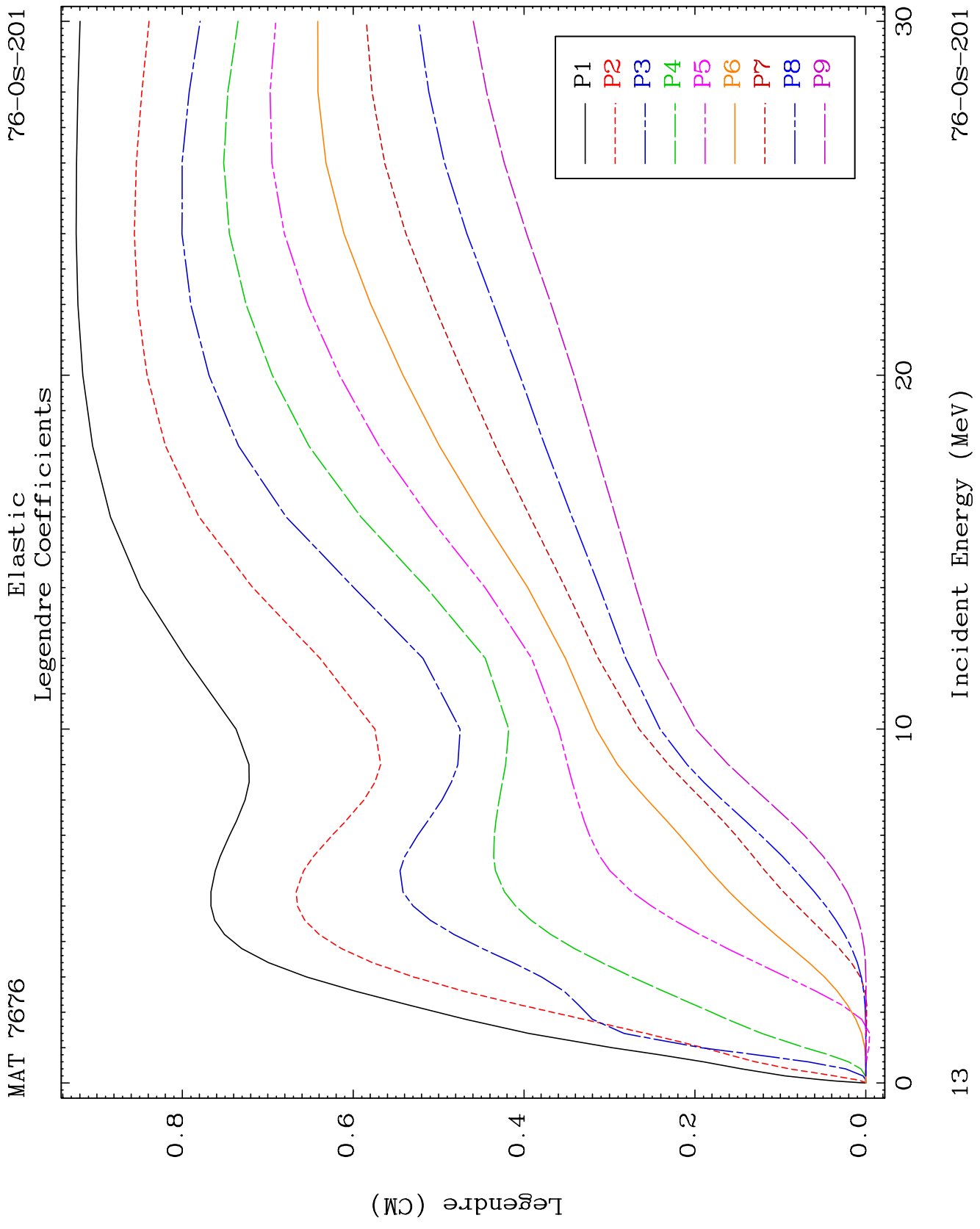
MAT 7676

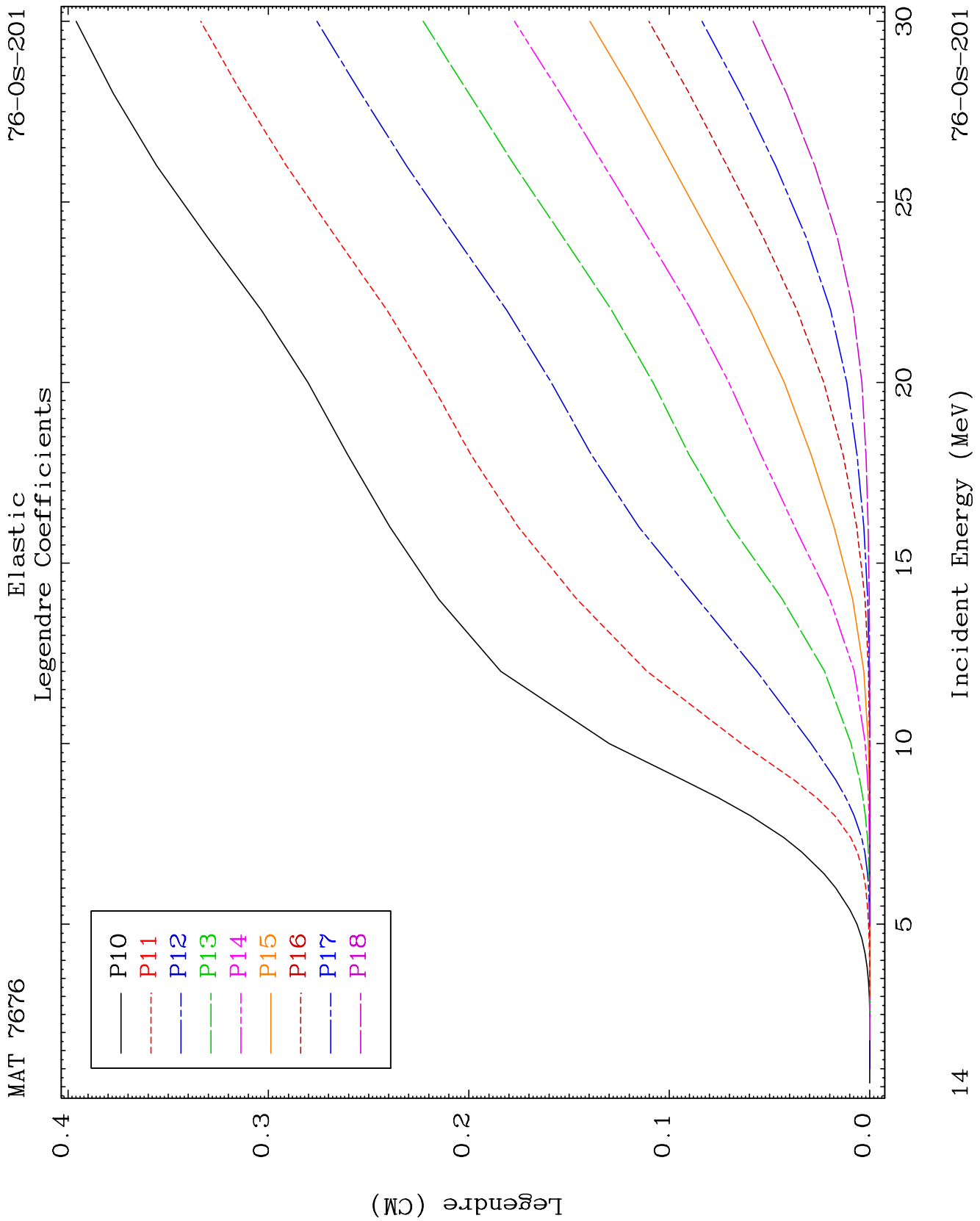
(n, α) Levels
293 Kelvin Cross Sections

76-Os-201





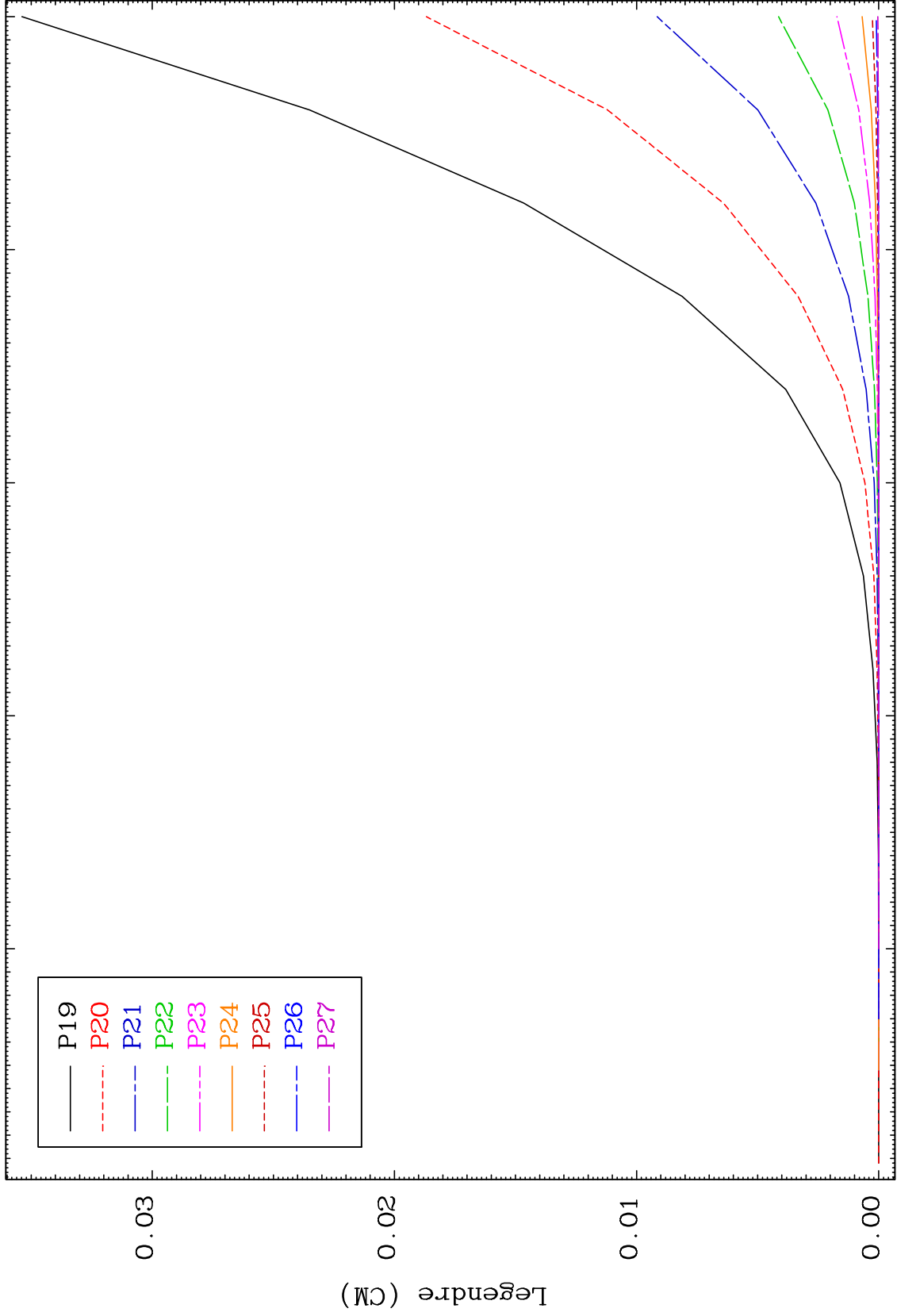




MAT 7676

Elastic Legendre Coefficients

76-0s-201



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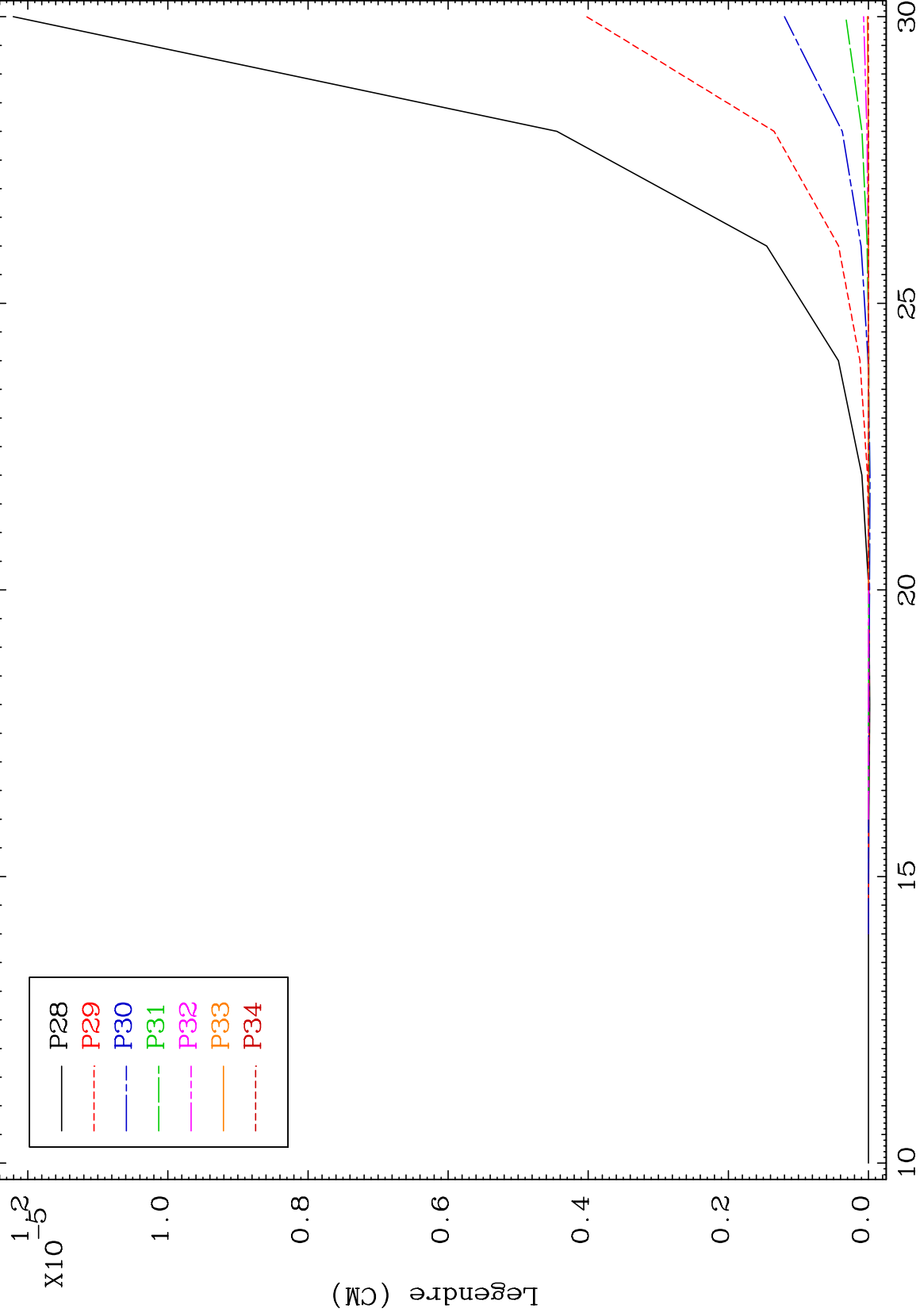
Incident Energy (MeV)

76-0s-201

MAT 7676

Elastic
Legendre Coefficients

76-0s-201



16

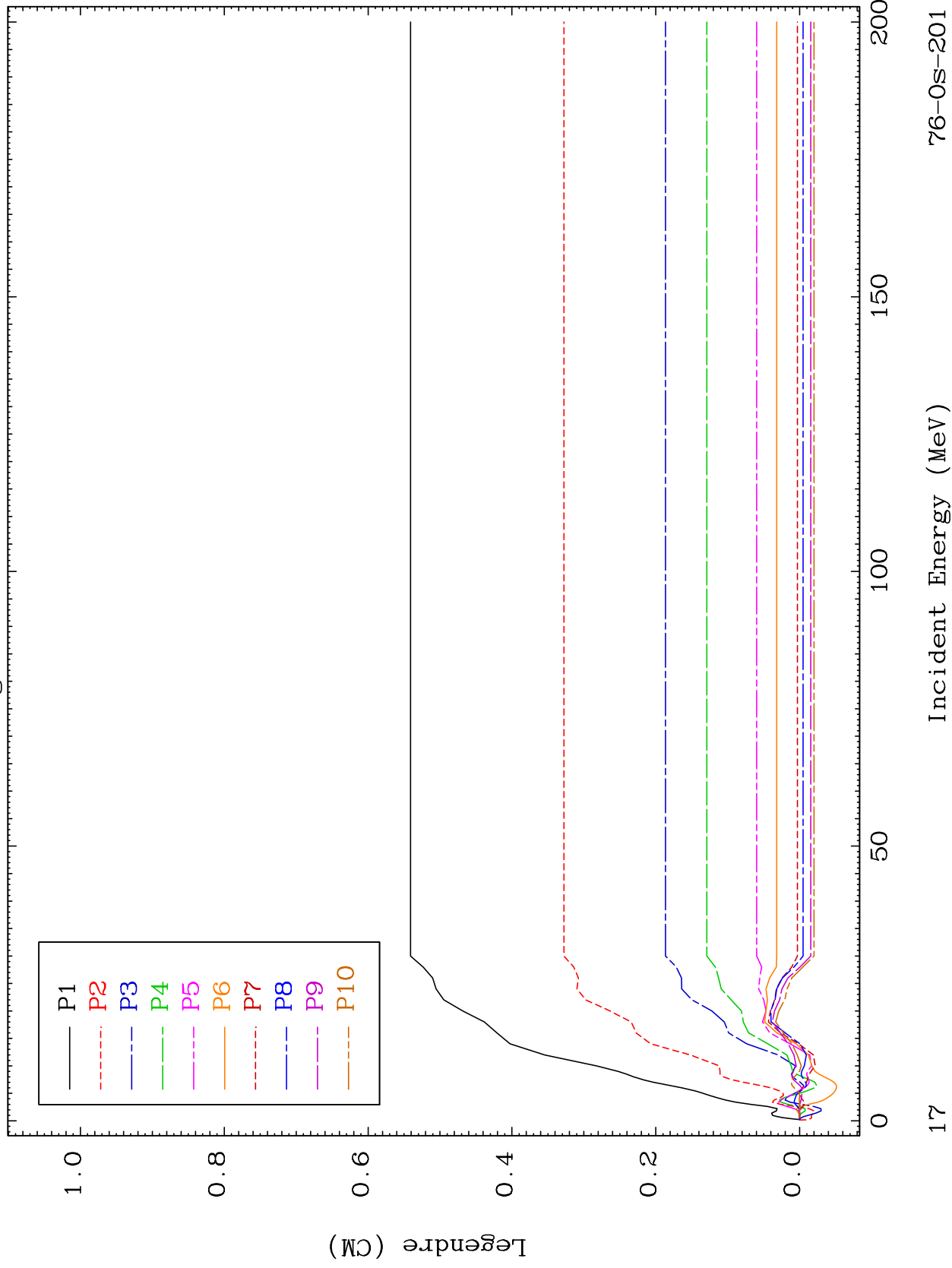
Incident Energy (MeV)

76-0s-201

MAT 7676

MT= 51 (n,n') Level
Legendre Coefficients

76-0s-201



17

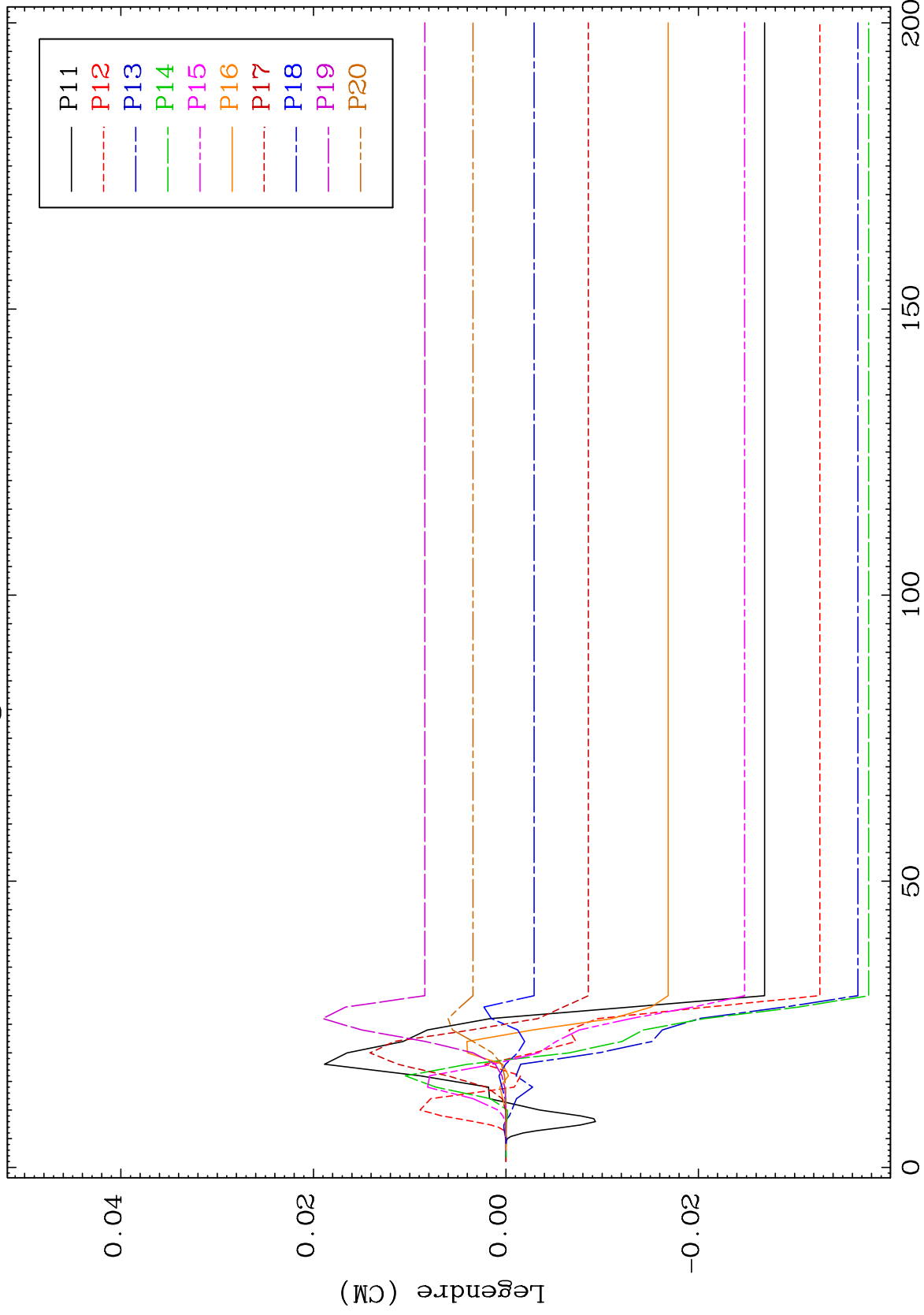
Incident Energy (MeV)

76-0s-201

MAT 7676

MT= 51 (n,n') Level
Legendre Coefficients

76-0s-201



18

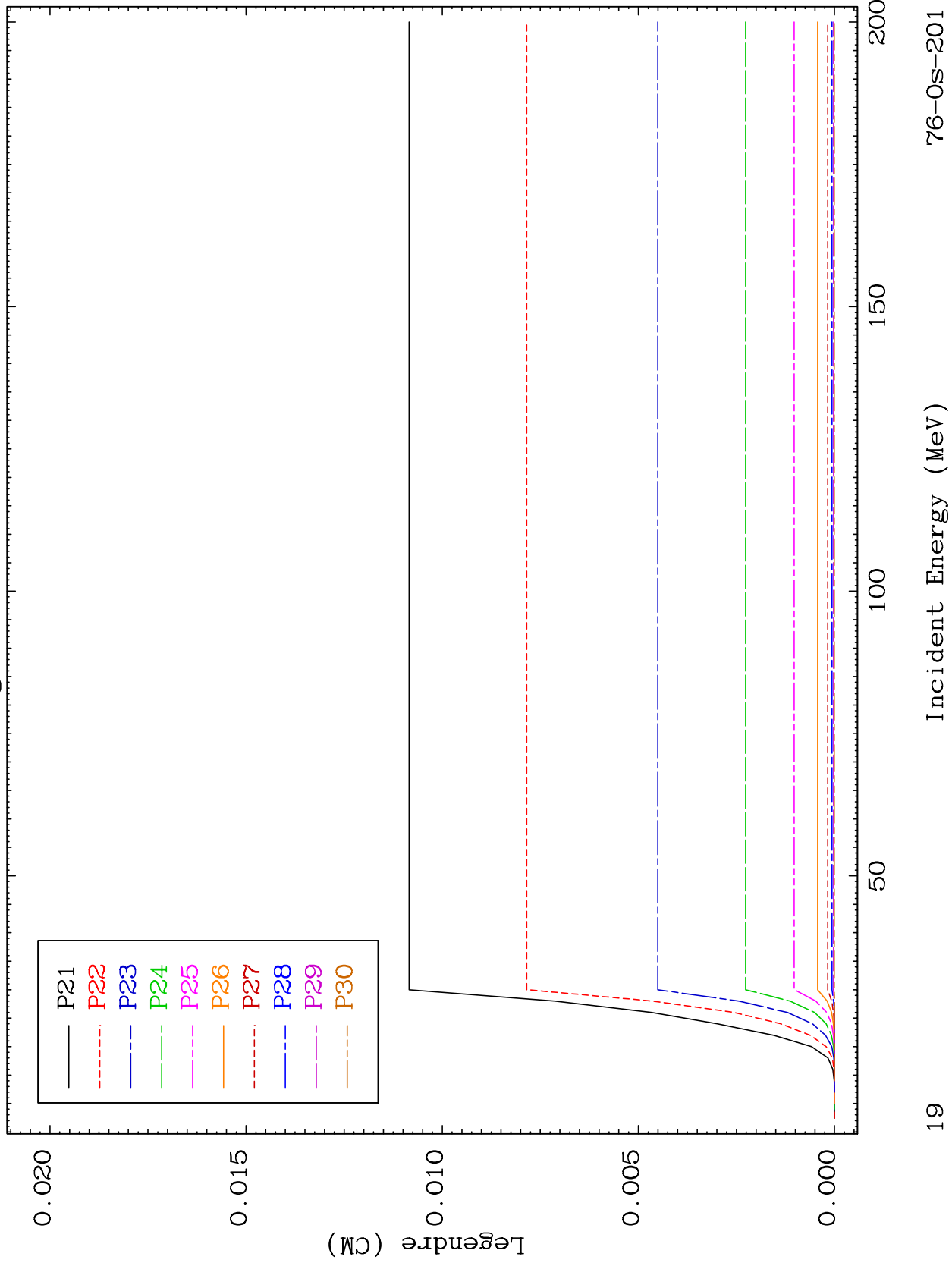
Incident Energy (MeV)

76-0s-201

MAT 7676

MT= 51 (n,n') Level
Legendre Coefficients

76-0s-201



19

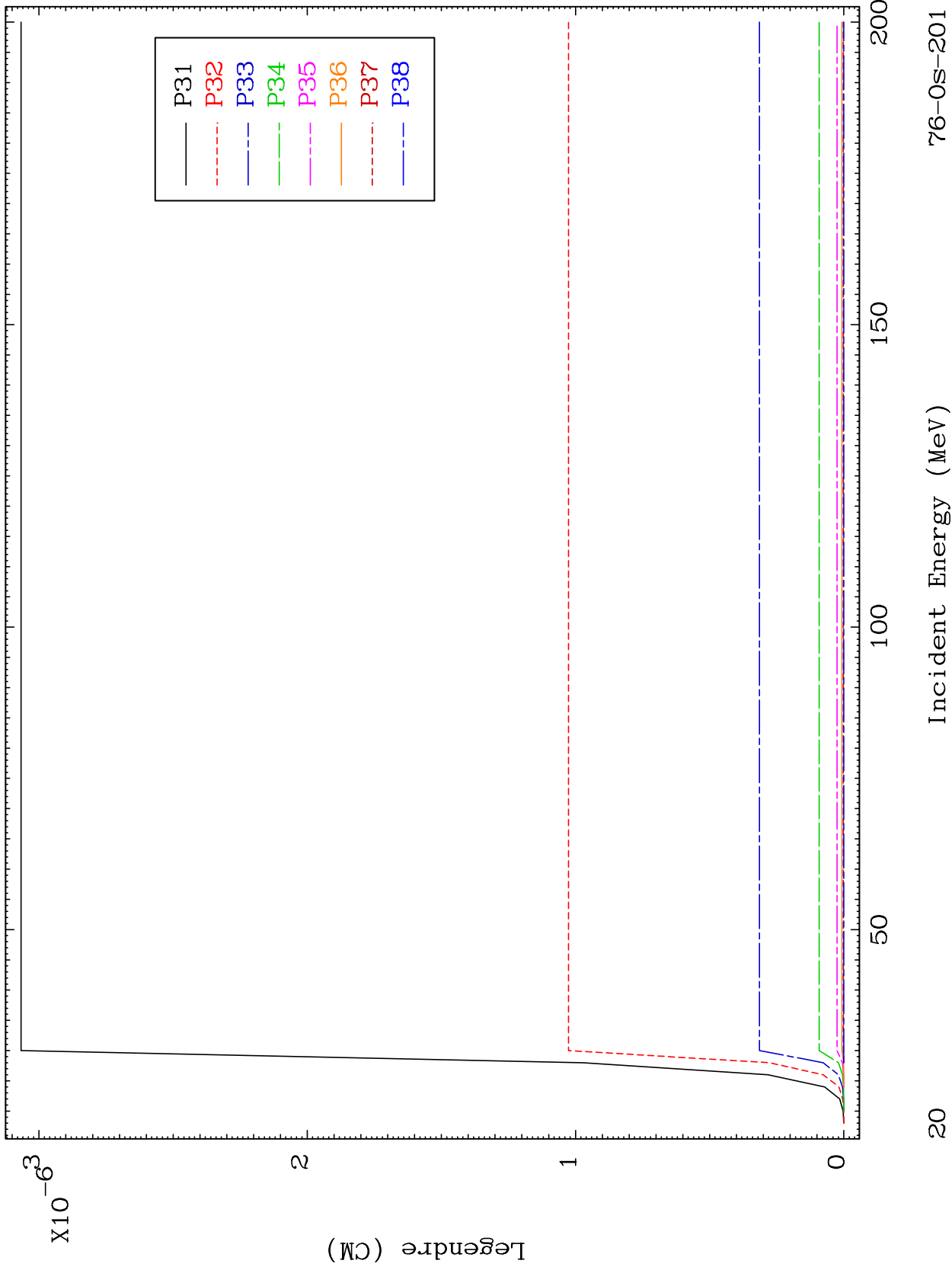
Incident Energy (MeV)

76-0s-201

MAT 7676

MT= 51 (n,n') Level
Legendre Coefficients

76-0s-201



MAT 7676

Fission

76-0s-201

Radionuclide Production Cross Section

