

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

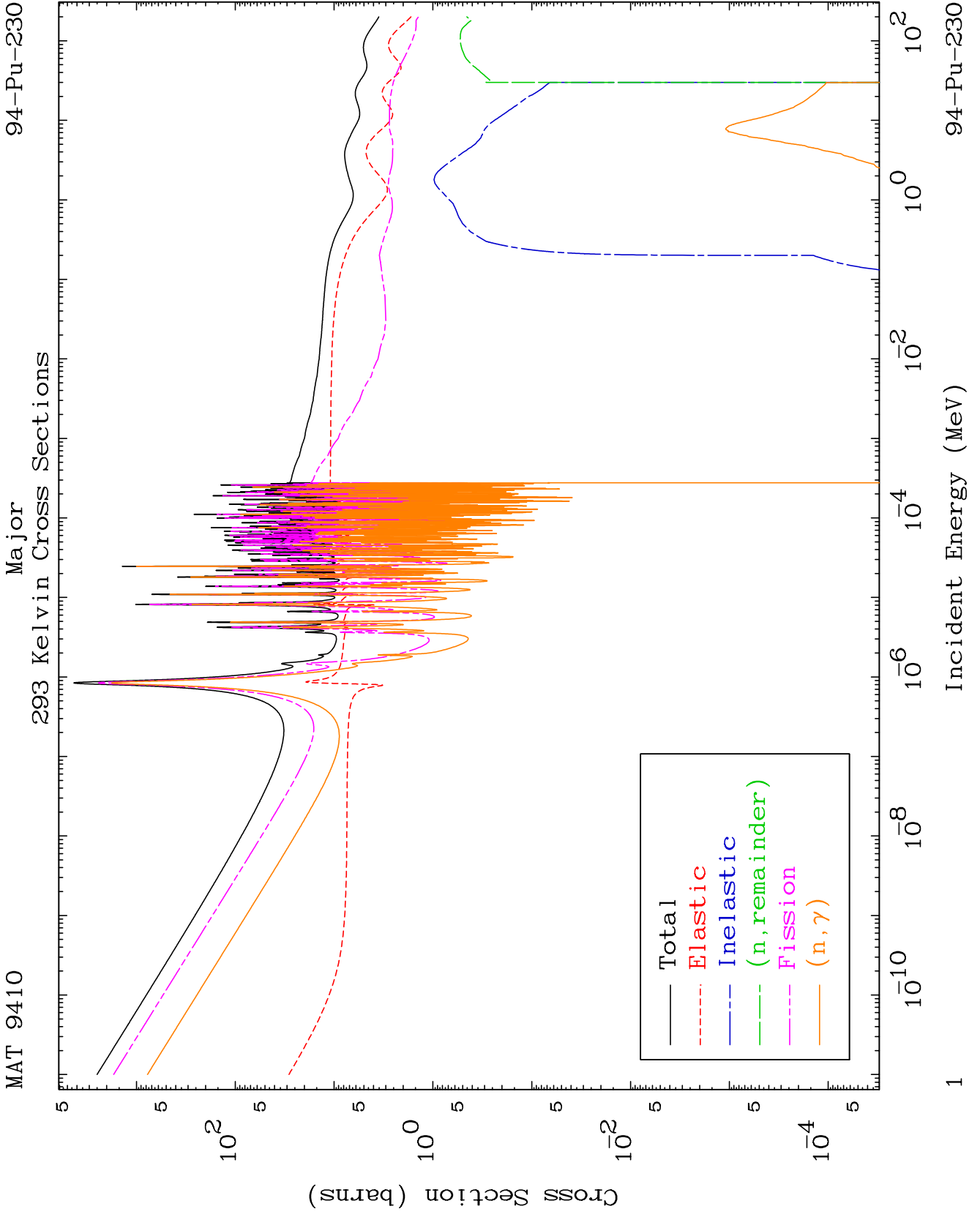
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Web:redcullen1.net/HOMEPAGE.NEW

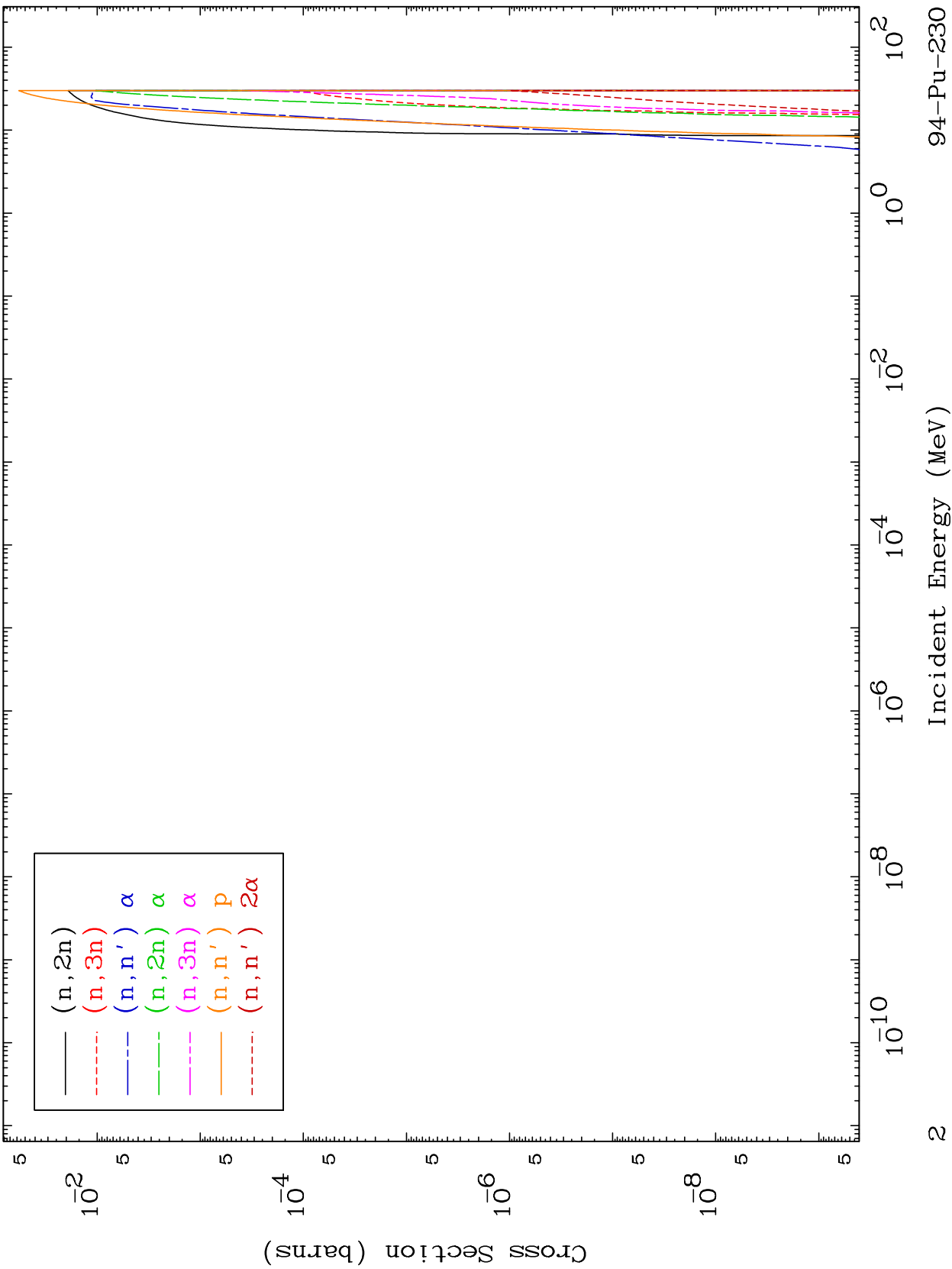
Press Mouse Button to Start



MAT 9410

Neutron Production
293 Kelvin Cross Sections

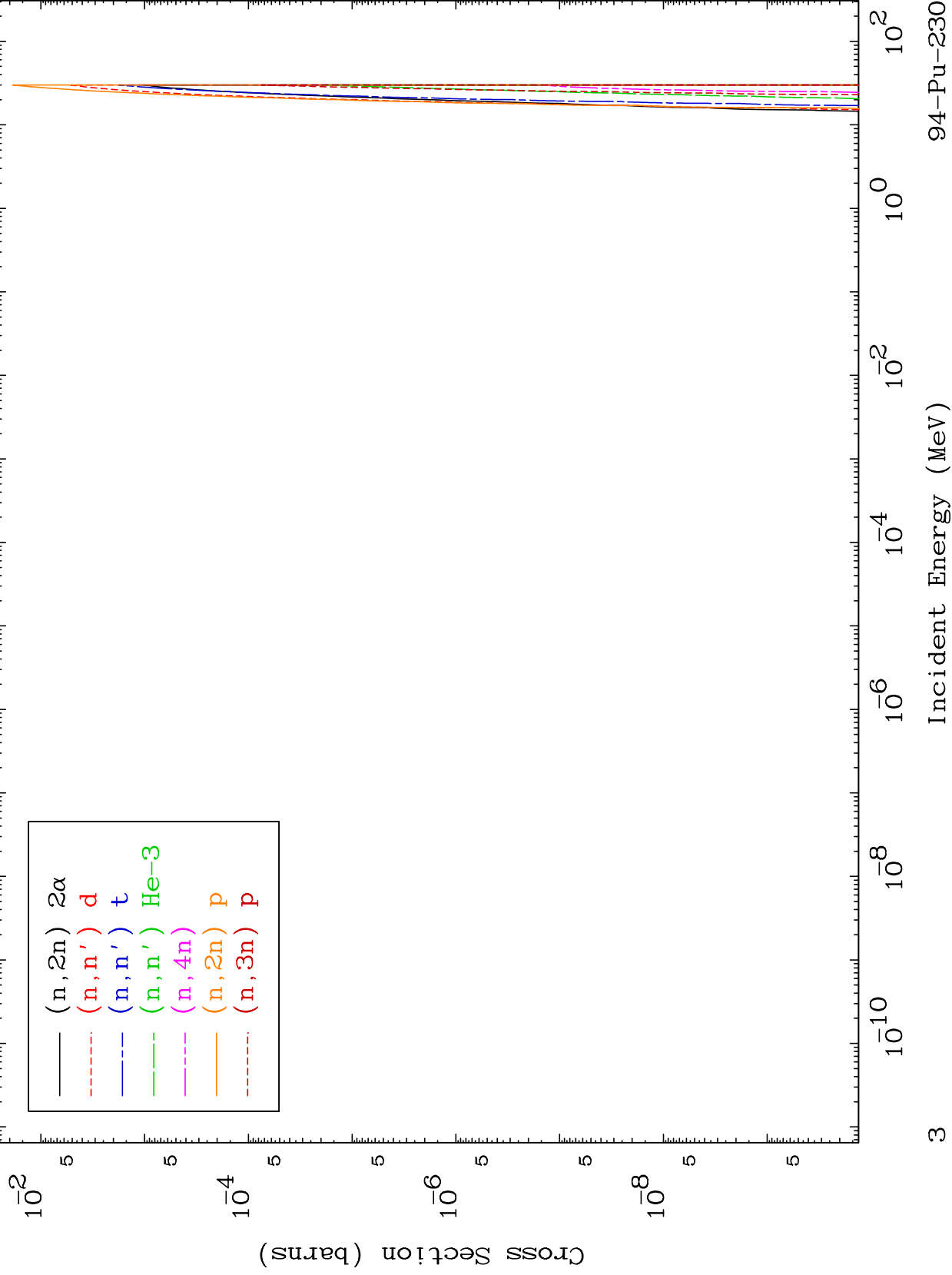
94-Pu-230



MAT 9410

Neutron Production
293 Kelvin Cross Sections

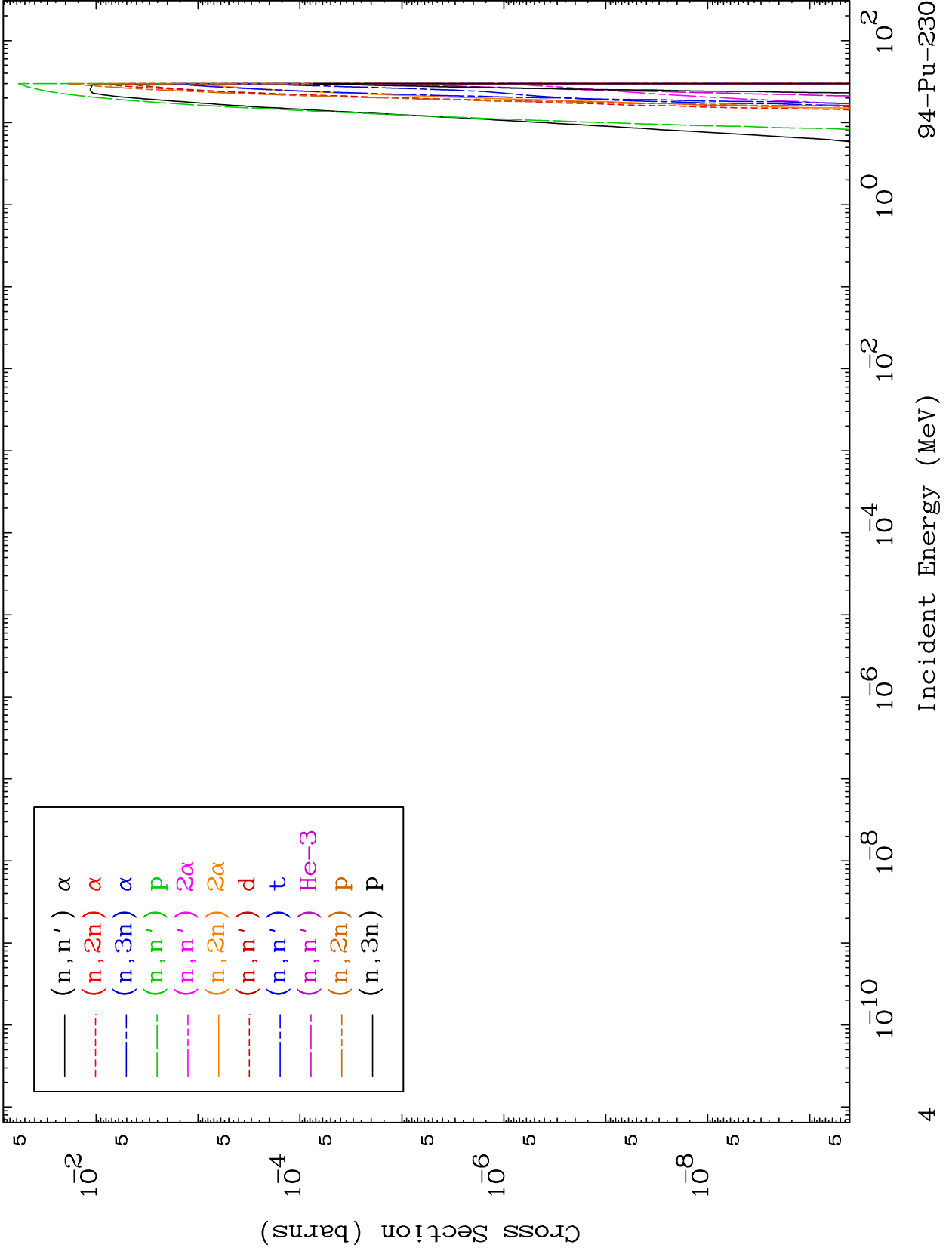
94-Pu-230



MAT 9410

Charged Particle
293 Kelvin Cross Sections

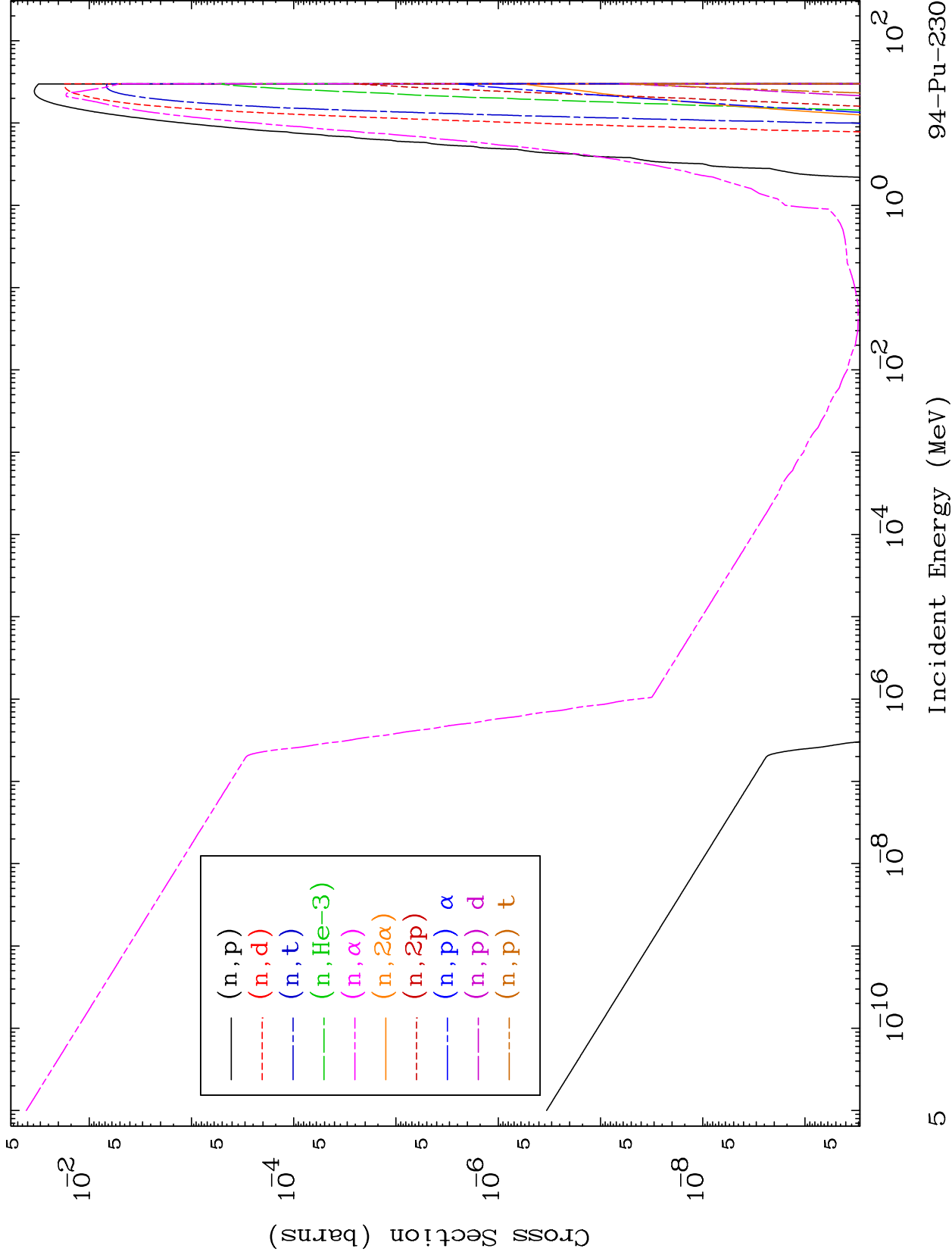
94-Pu-230



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Charged Particle
293 Kelvin Cross Sections

94-Pu-230

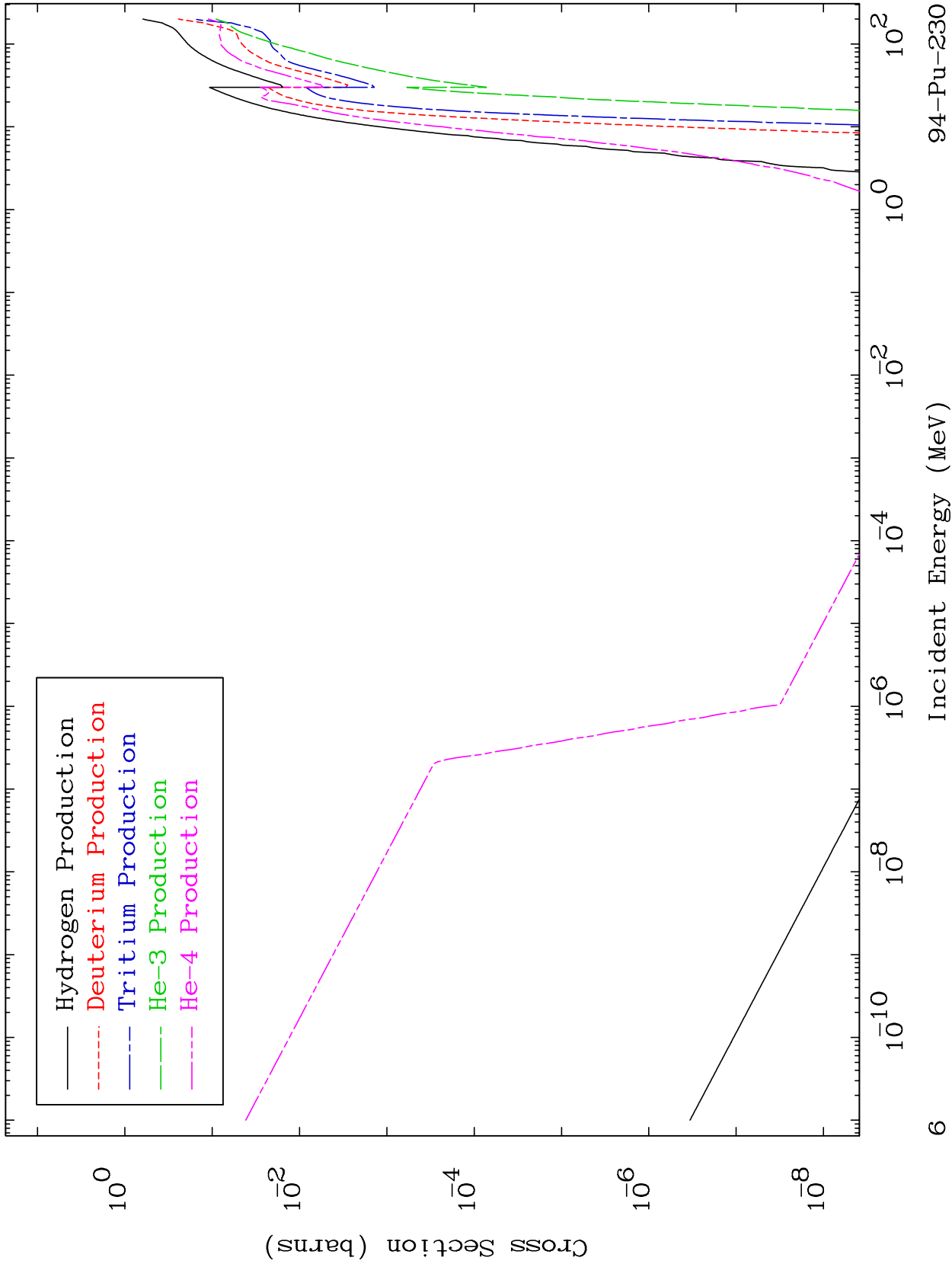


5

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Particle Production
293 Kelvin Cross Sections

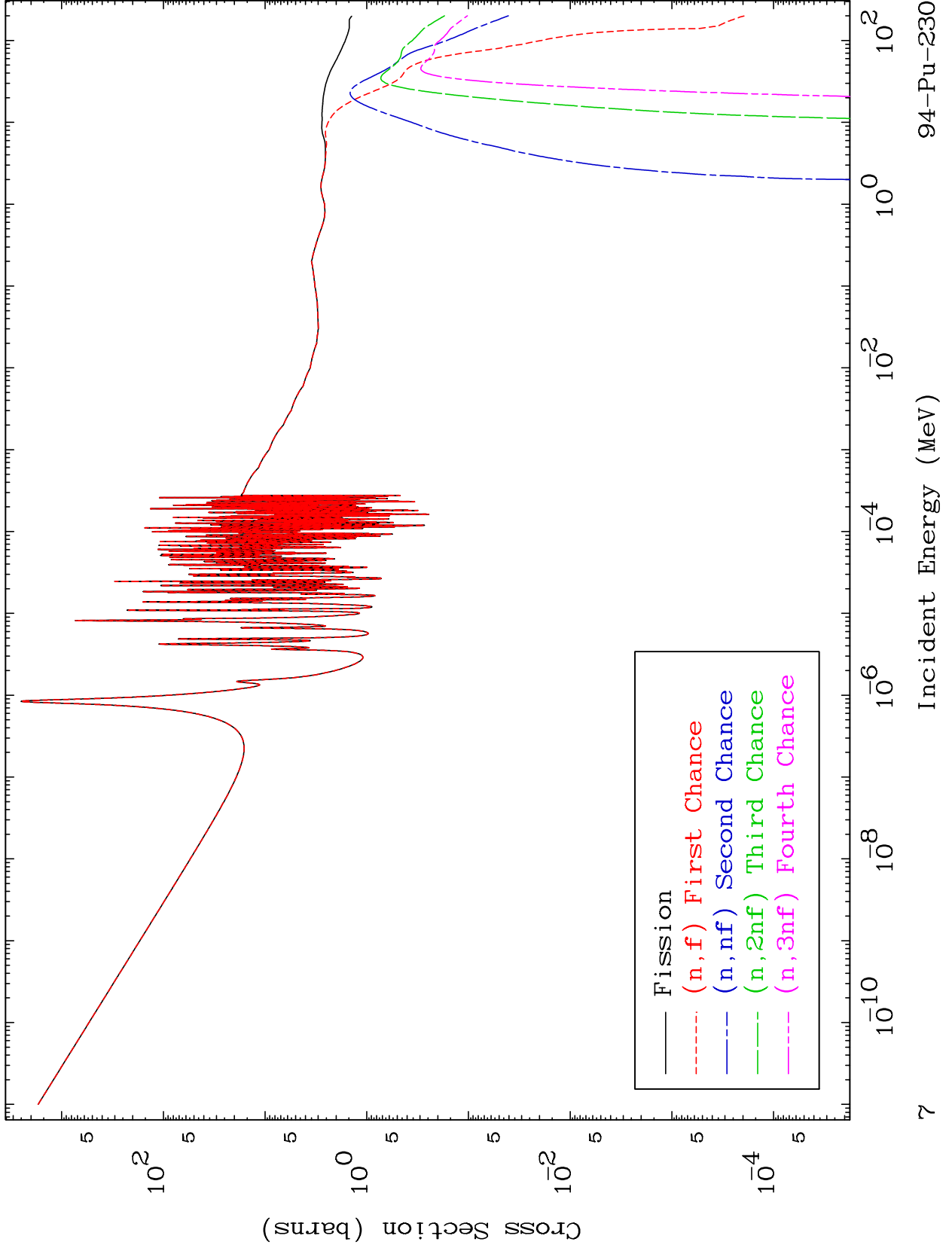
94-Pu-230



MAT 9410

Fission
293 Kelvin Cross Sections

94-Pu-230

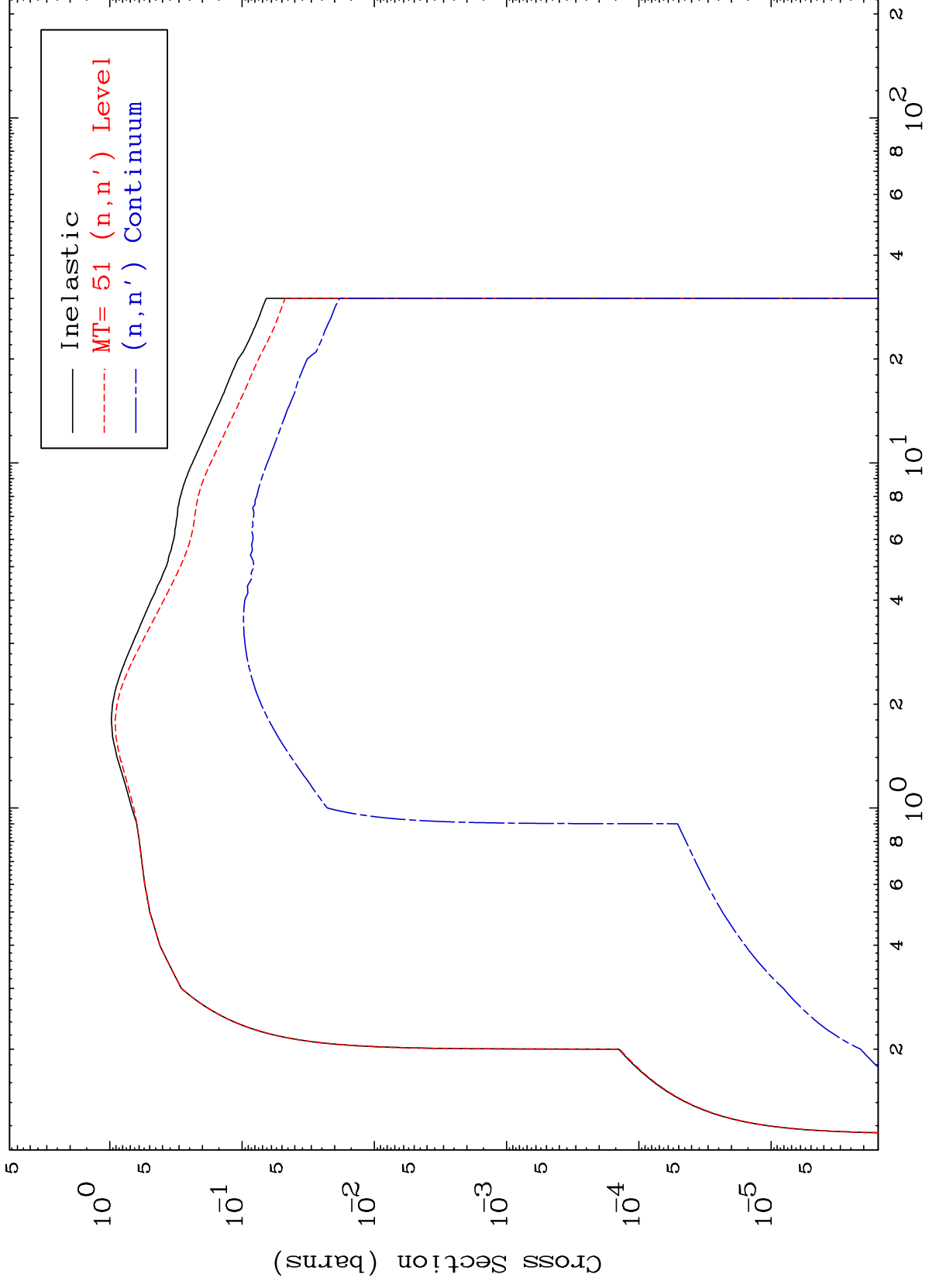


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(n,n') Level

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293 Kelvin Cross Sections

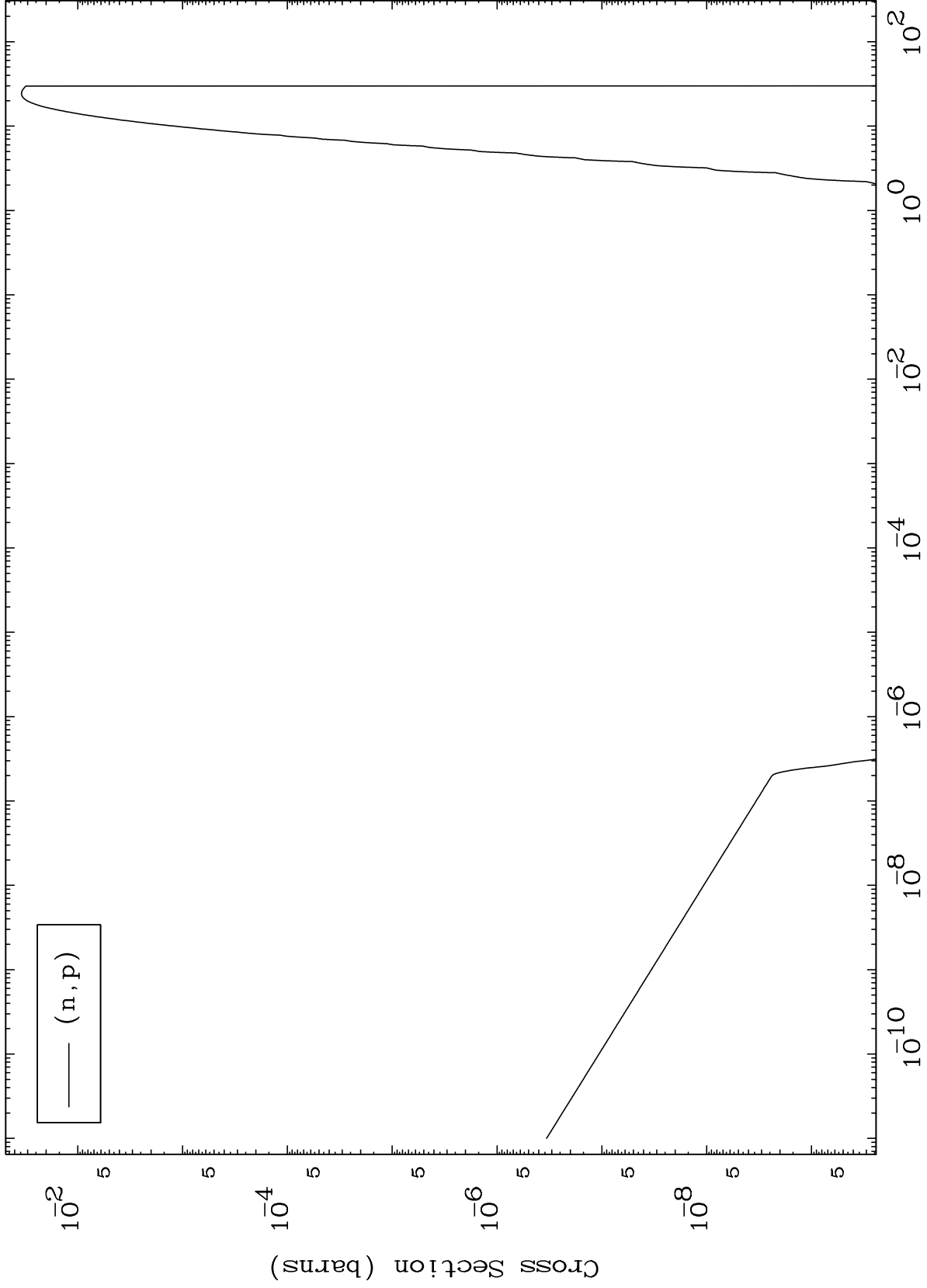


— Inelastic
- - - MT=51 (n,n') Level
- - - (n,n') Continuum

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(n,p) Levels
293 Kelvin Cross Sections

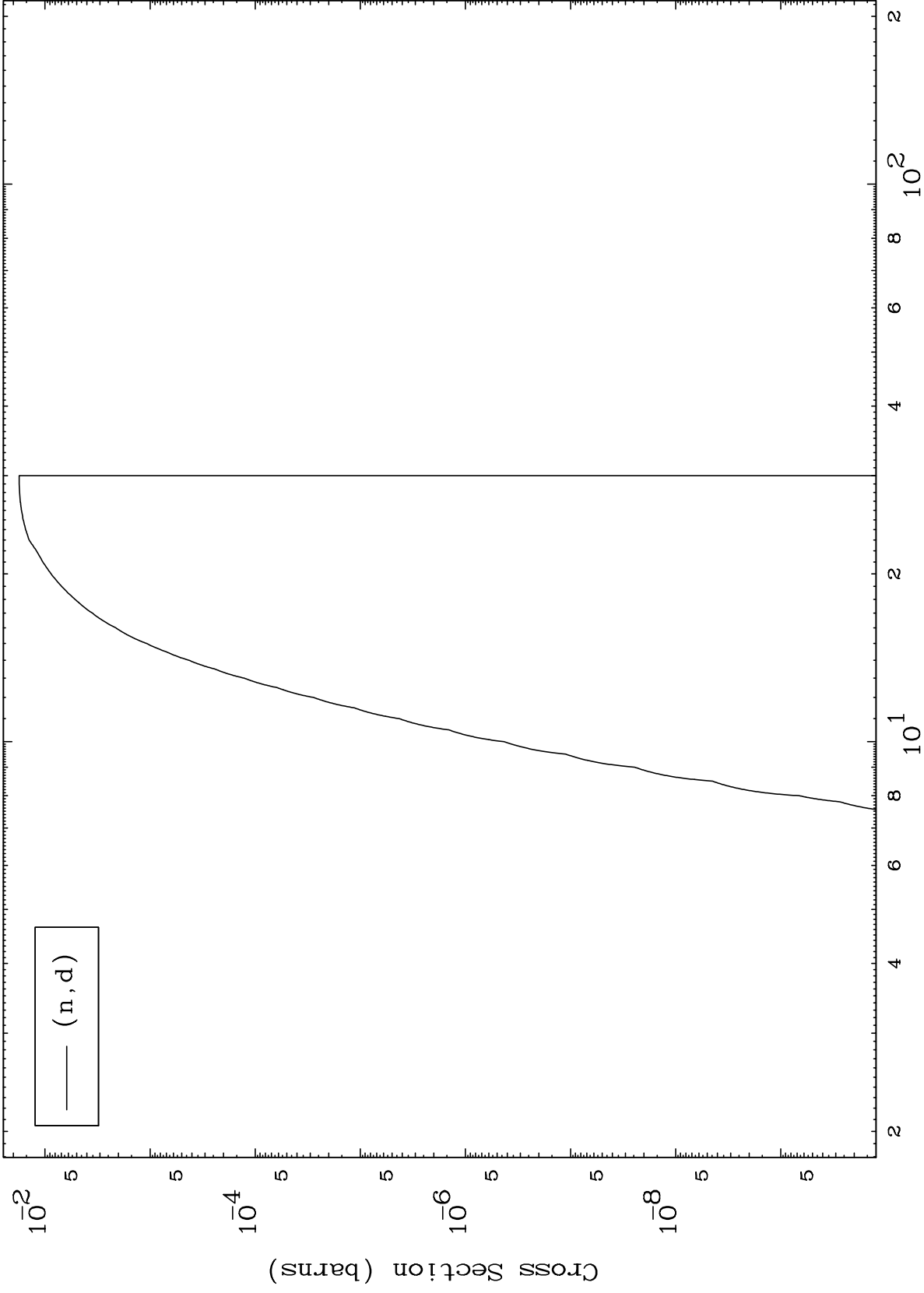
94-Pu-230



MAT 9410

94-Pu-230

(n,d) Levels
293 Kelvin Cross Sections



94-Pu-230

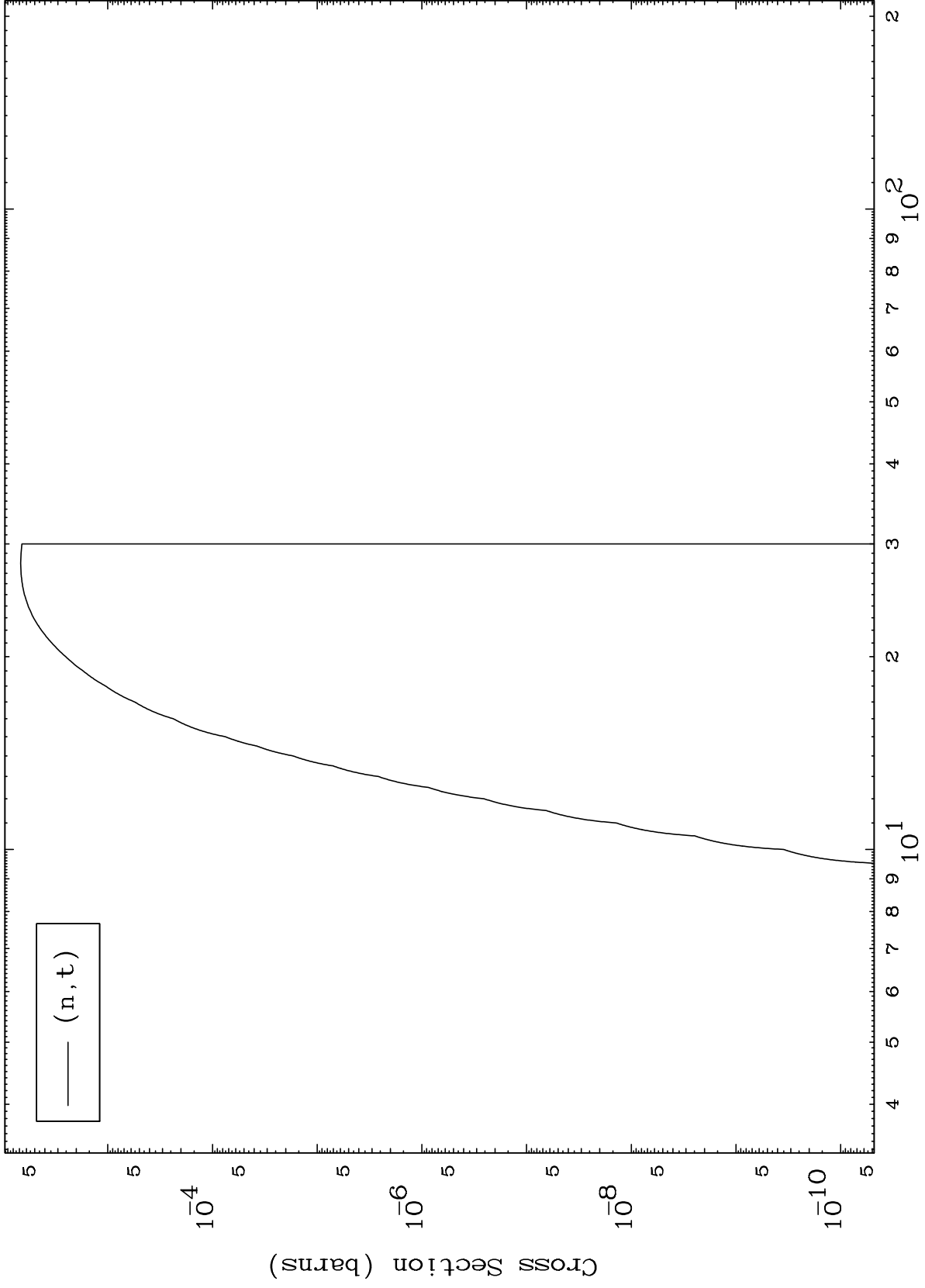
Incident Energy (MeV)

10

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(n,t) Levels
293 Kelvin Cross Sections

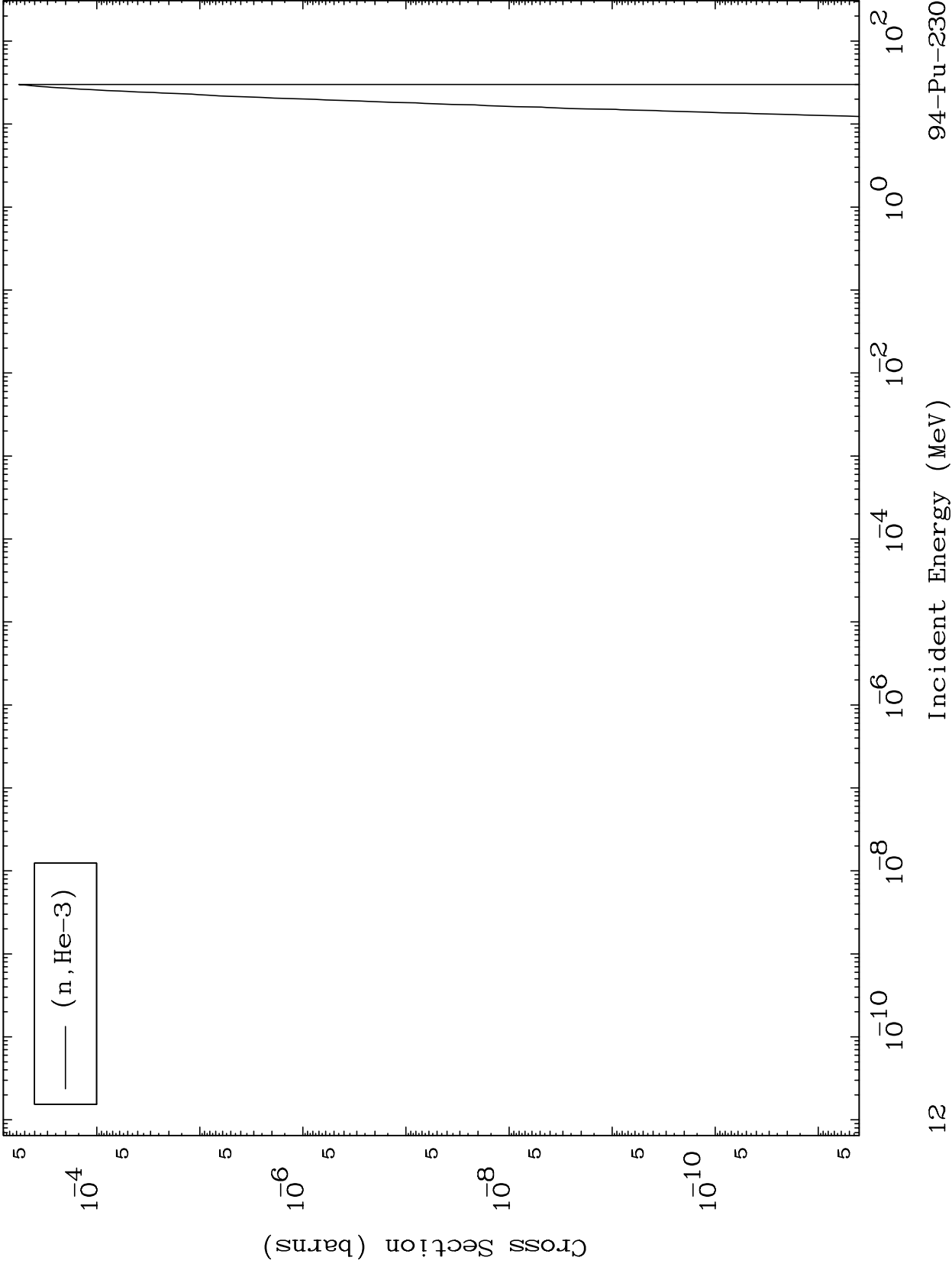
94-Pu-230



MAT 9410

(n,He3) Levels
293 Kelvin Cross Sections

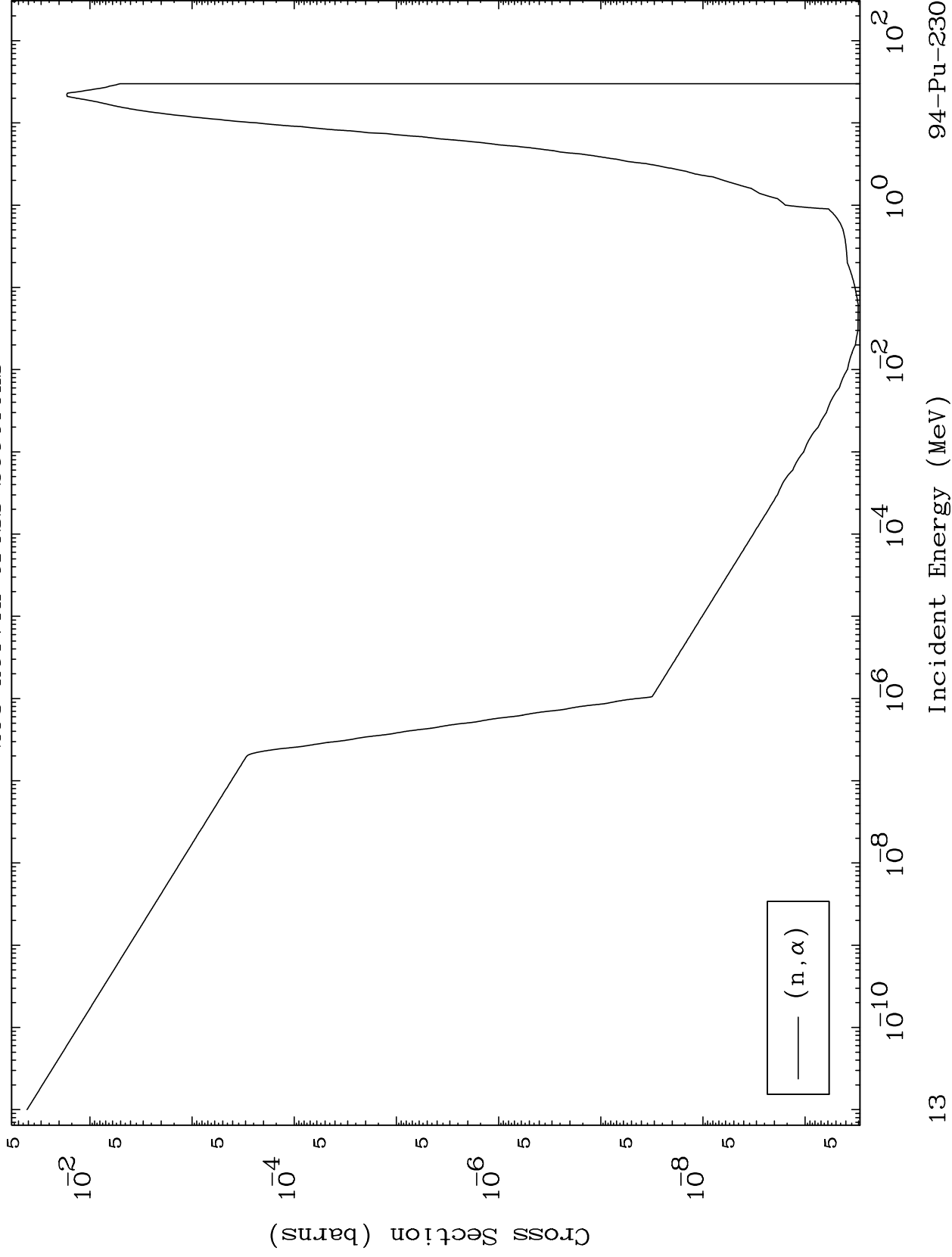
94-Pu-230



MAT 9410

(n,α) Levels
293 Kelvin Cross Sections

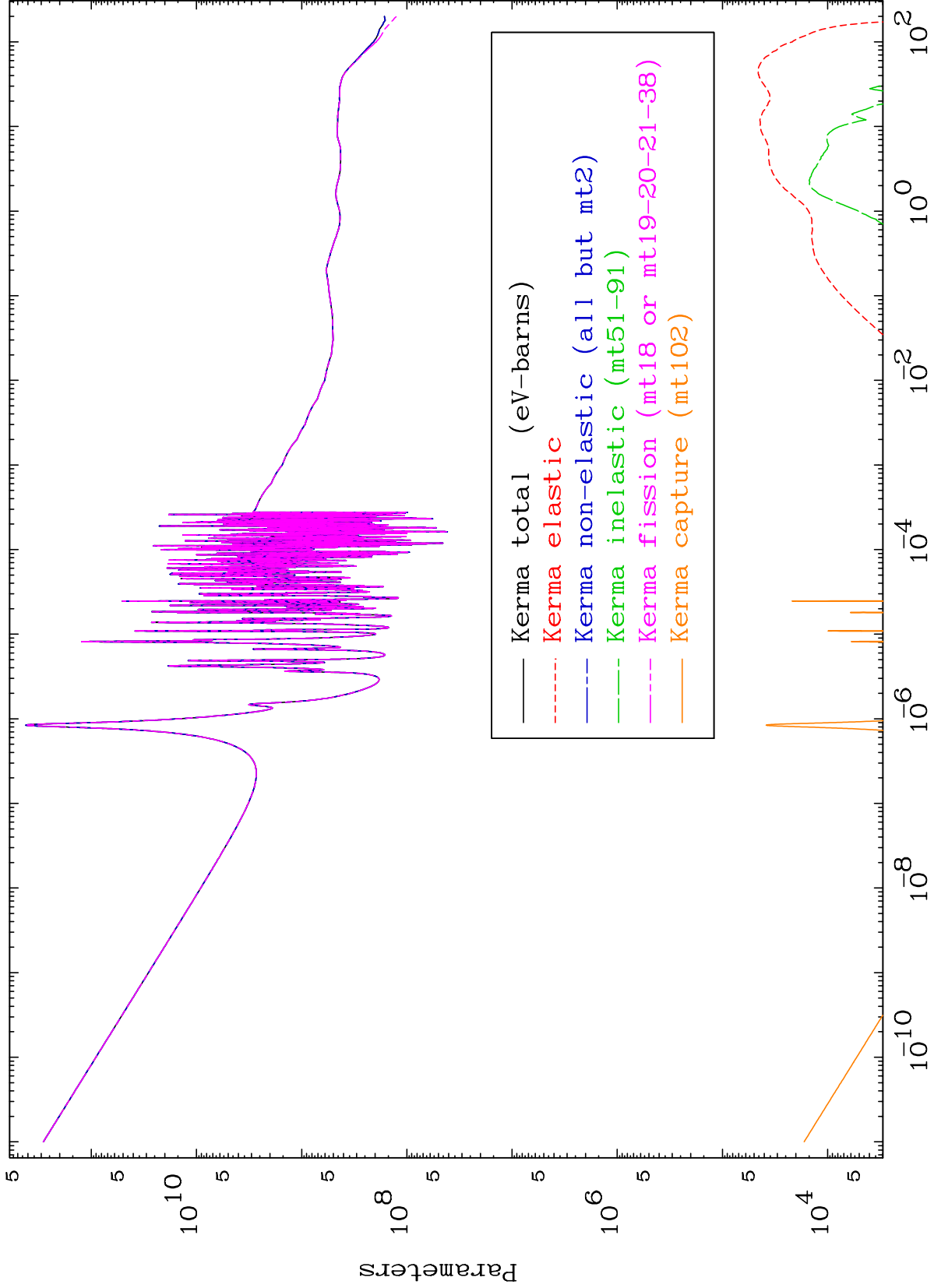
94-Pu-230

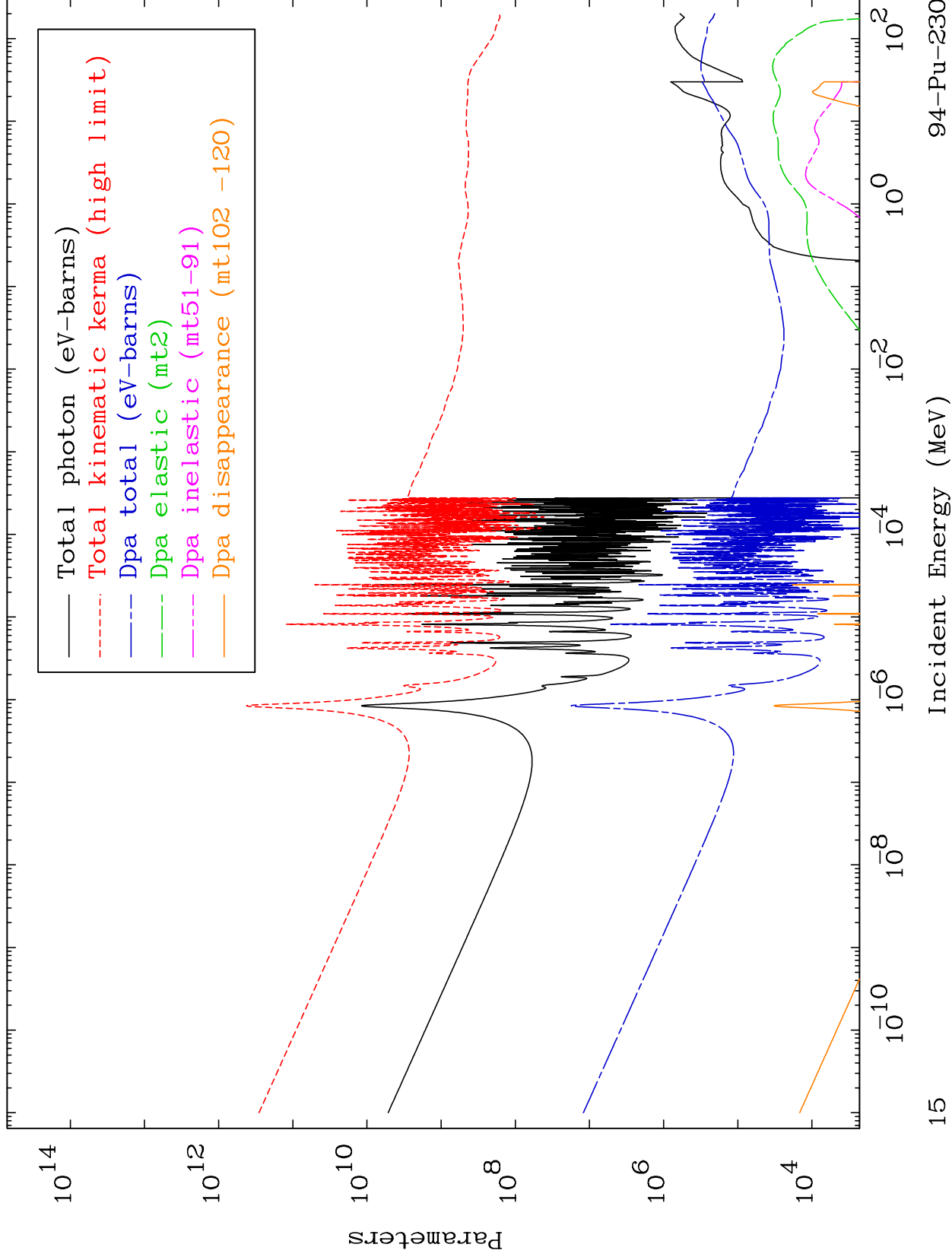


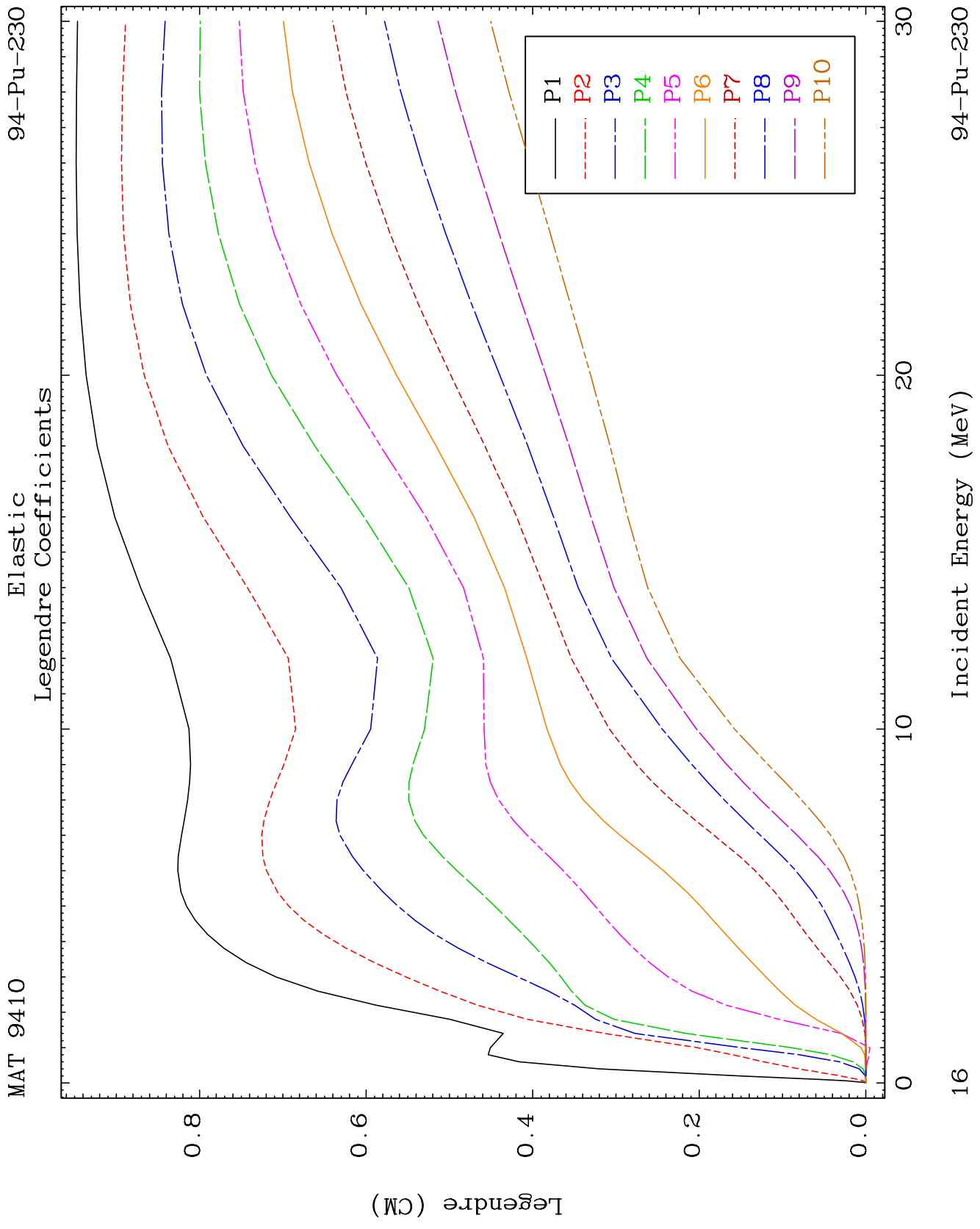
MAT 9410

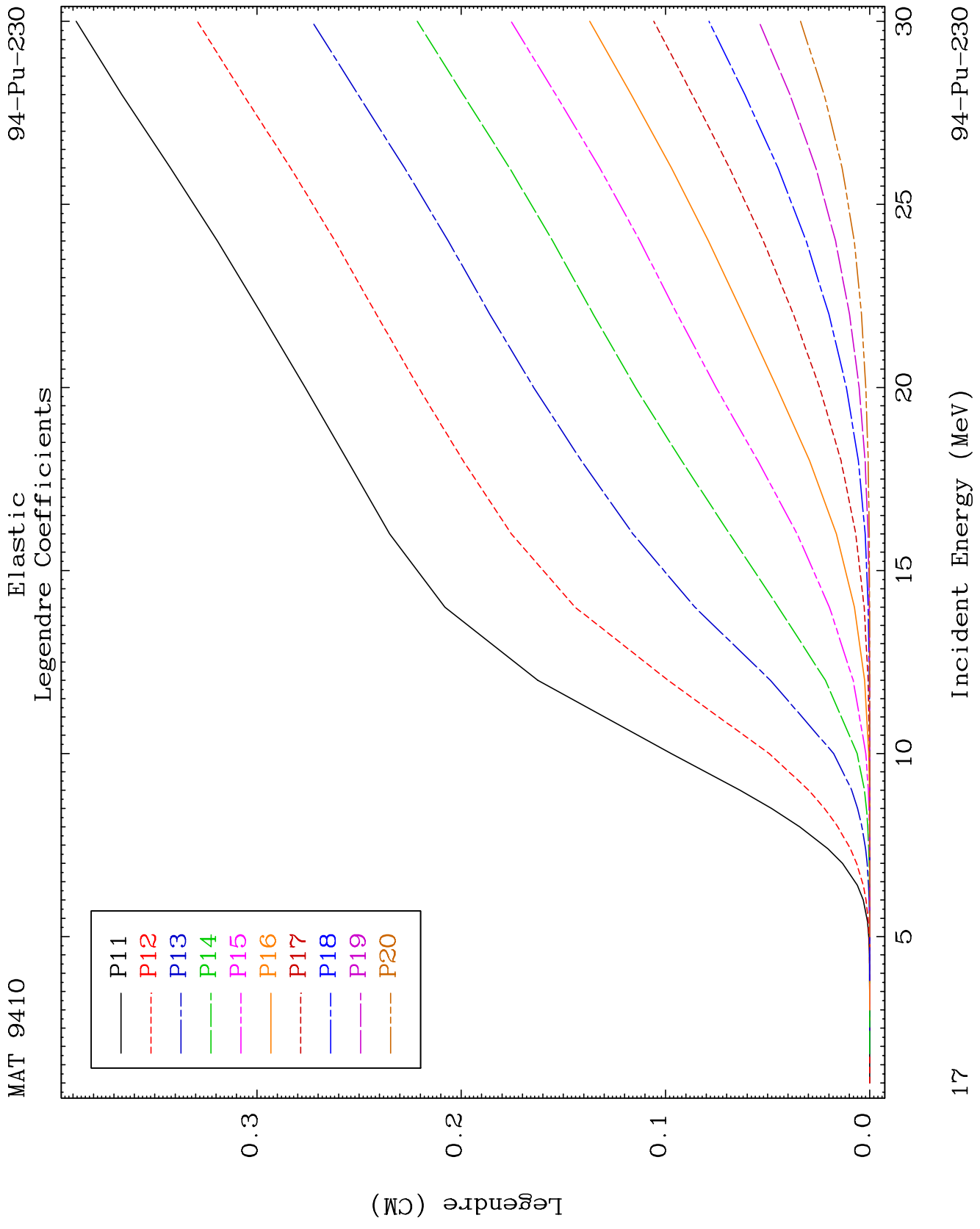
Energy Release
Parameters

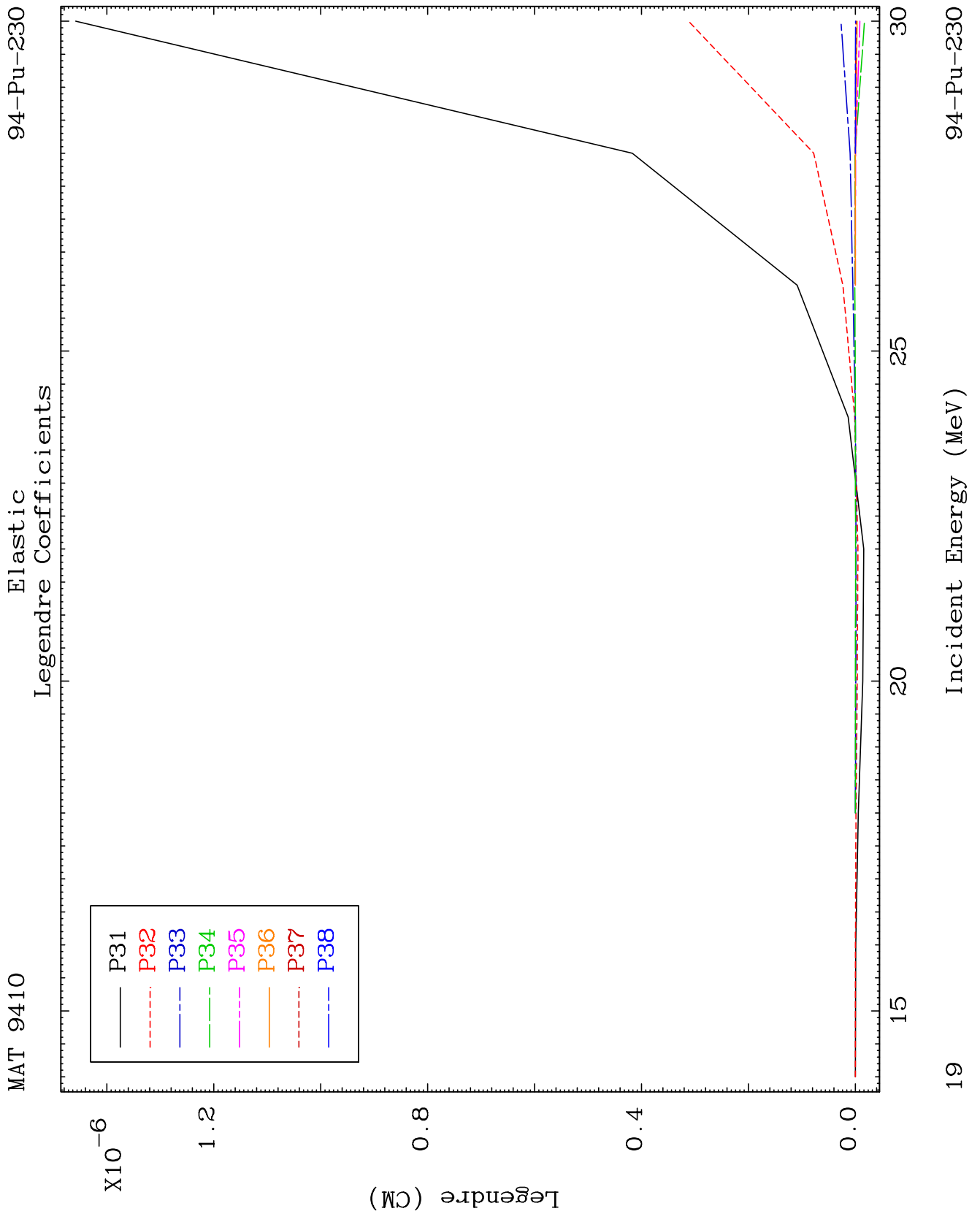
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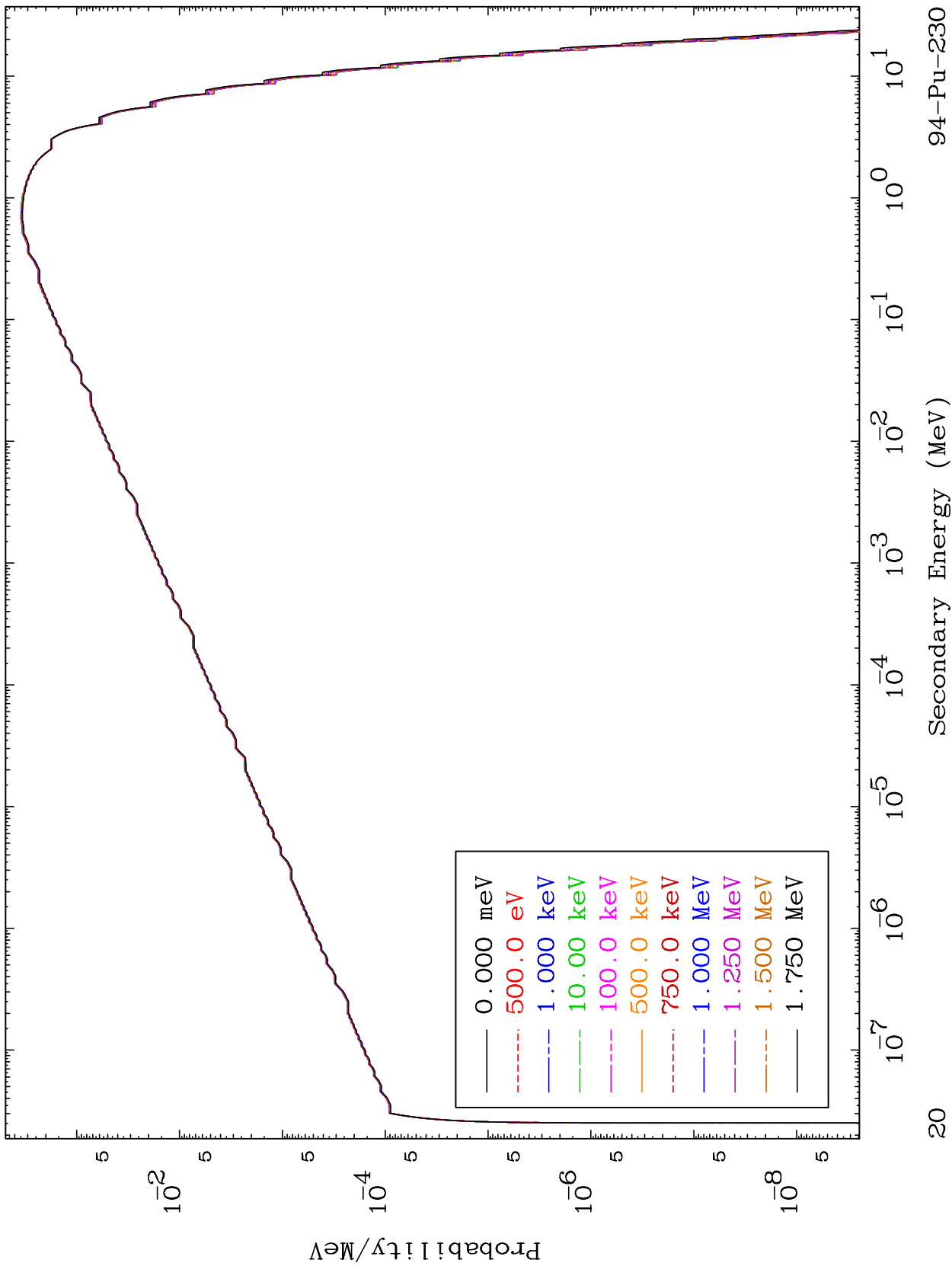




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Fission Energy Distribution

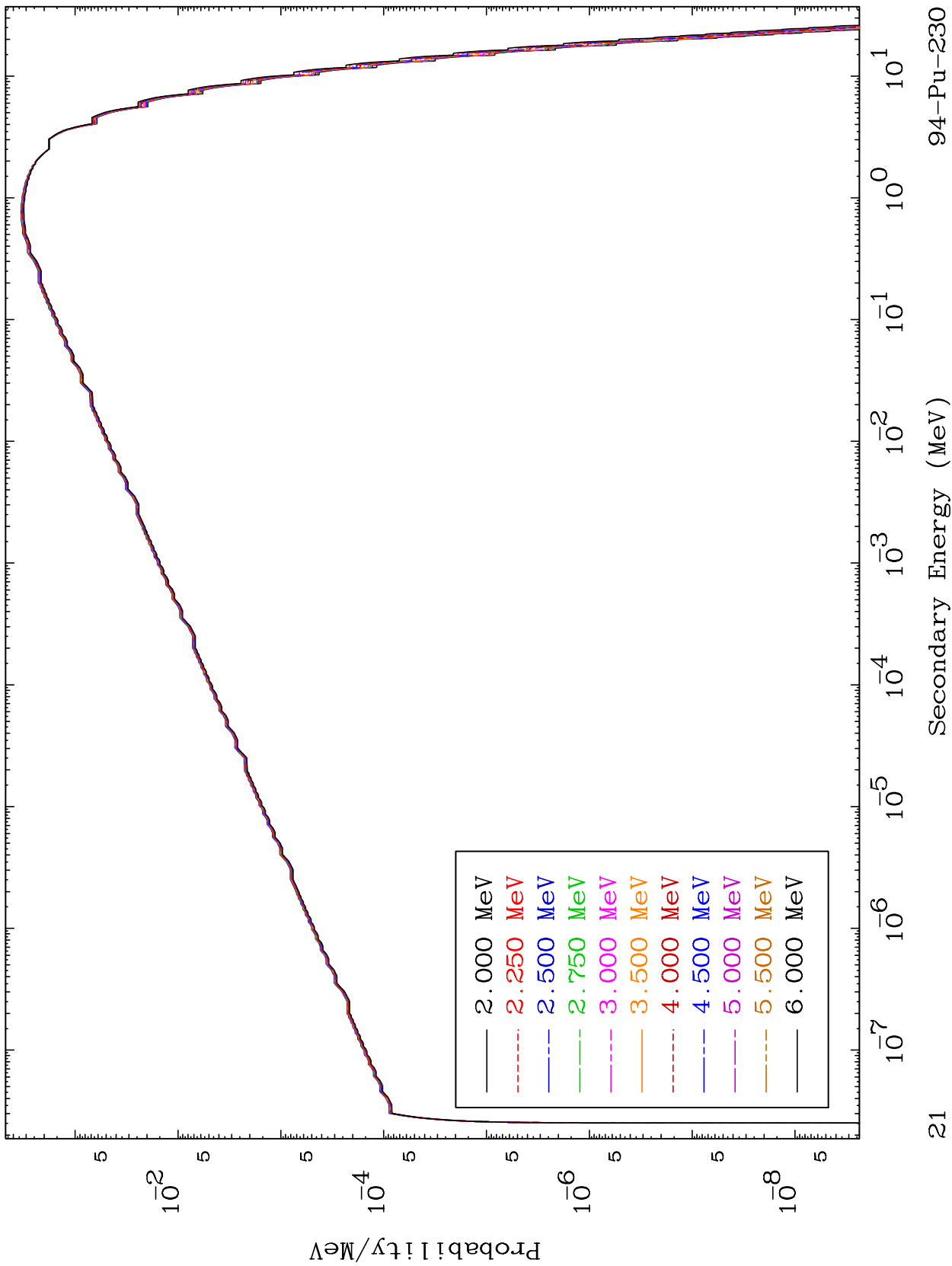
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Fission Energy Distribution

94-Pu-230

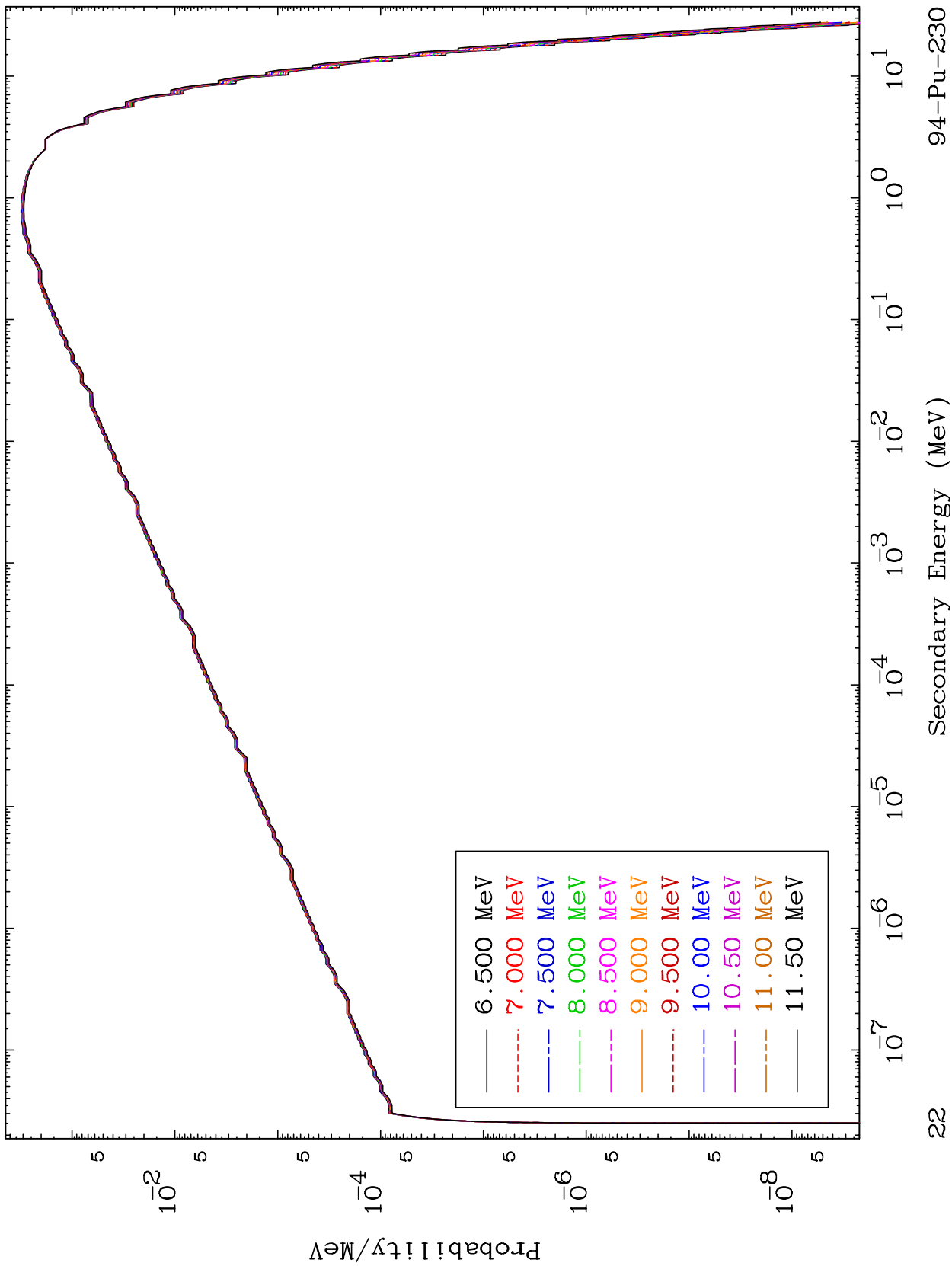


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MAT 9410

Fission Energy Distribution

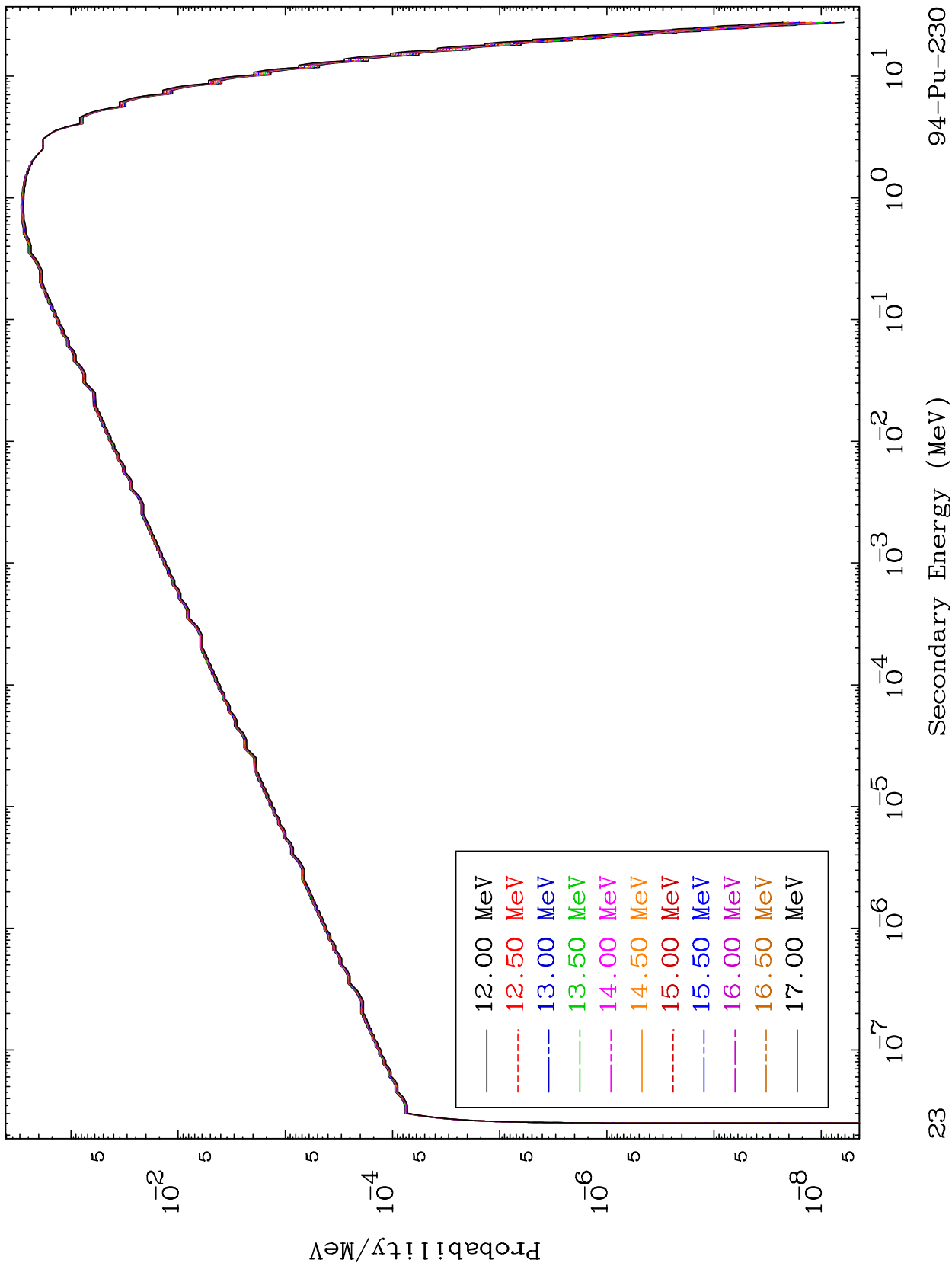
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MAT 9410

Fission Energy Distribution

94-Pu-230



94-Pu-230

MAT 9410

Fission Energy Distribution

94-Pu-230

