

Program EVALPLOT
(Version 2017-1)

by

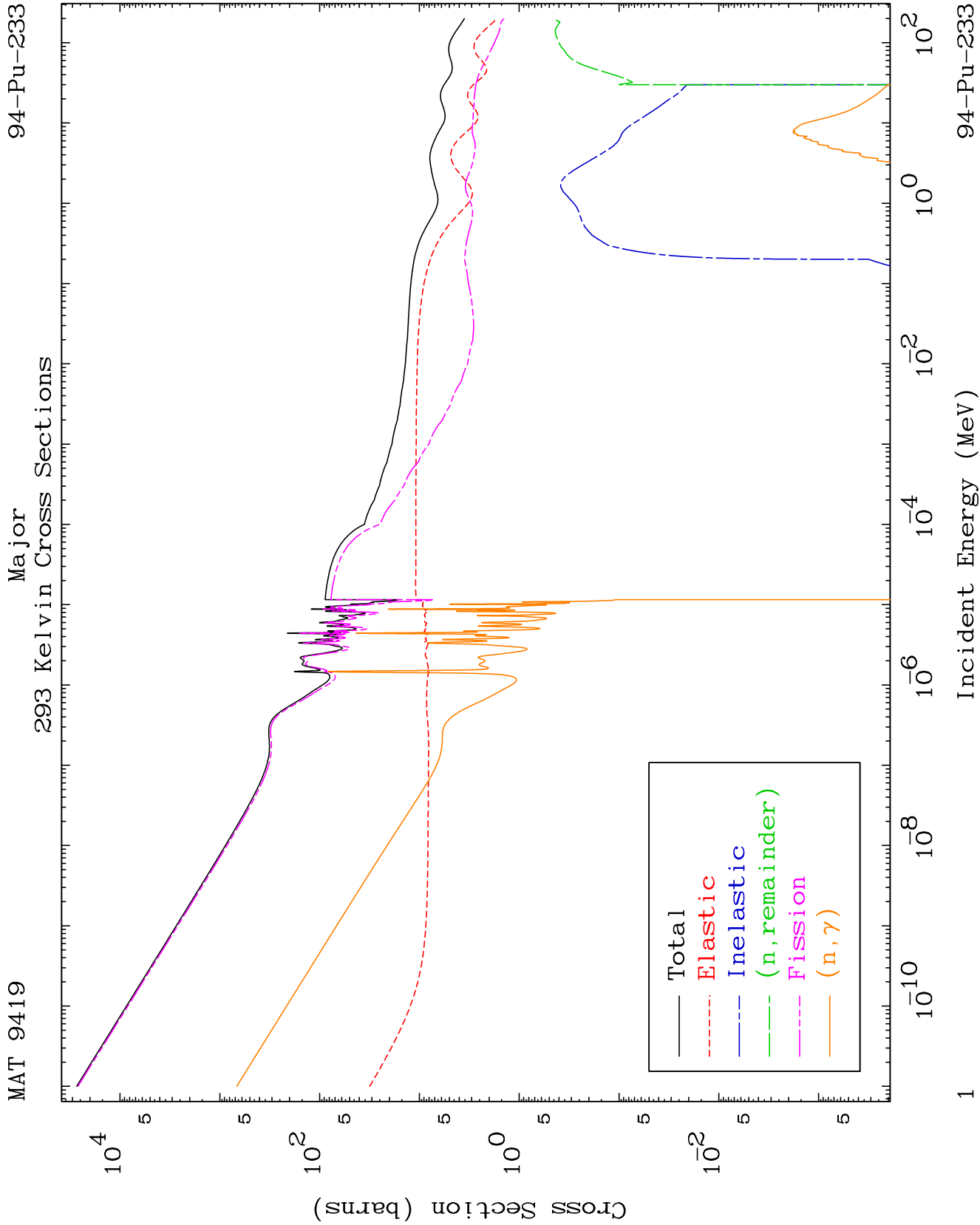
Dermott E. Cullen
(Present Contact Information)

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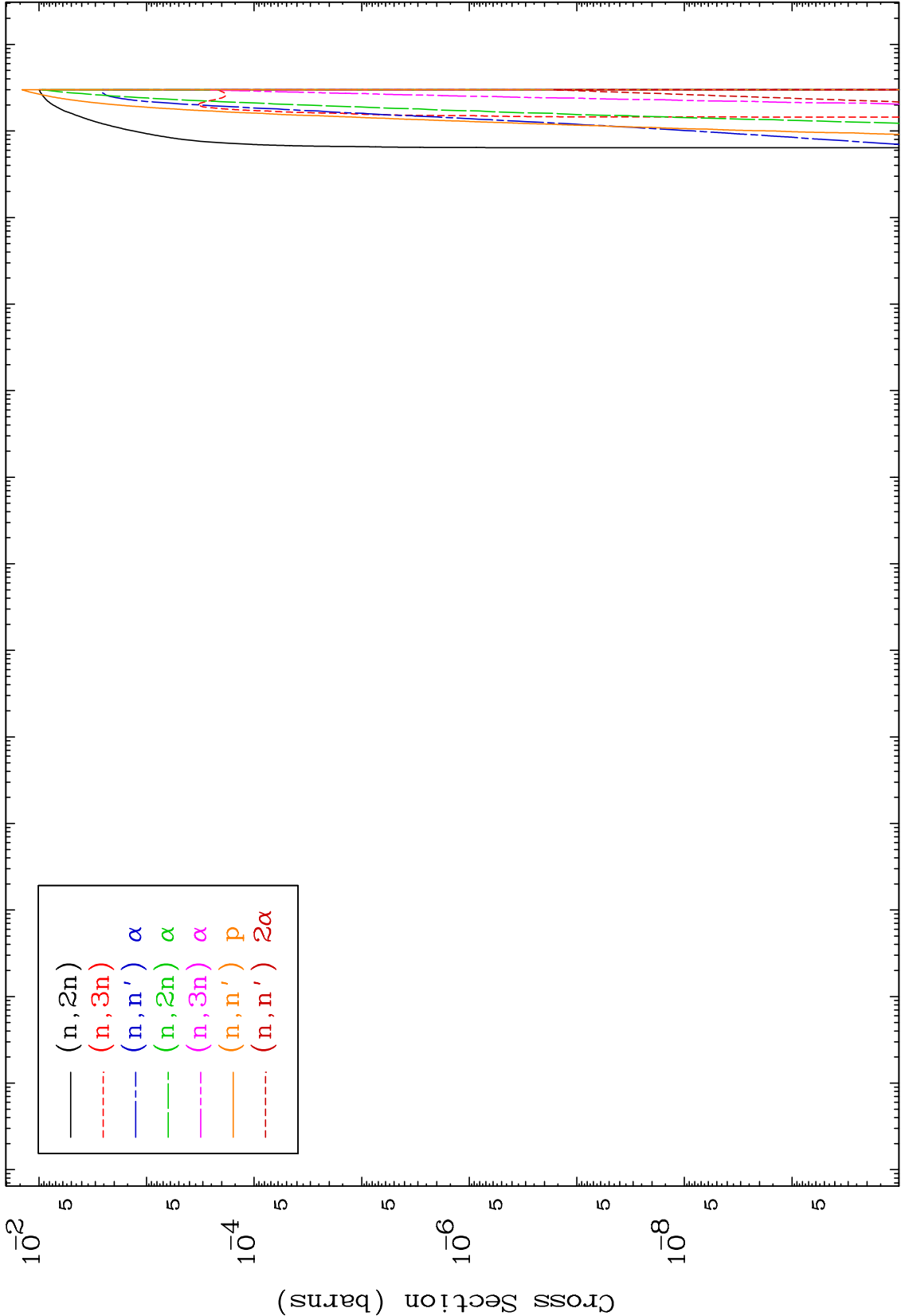
Press Mouse Button to Start

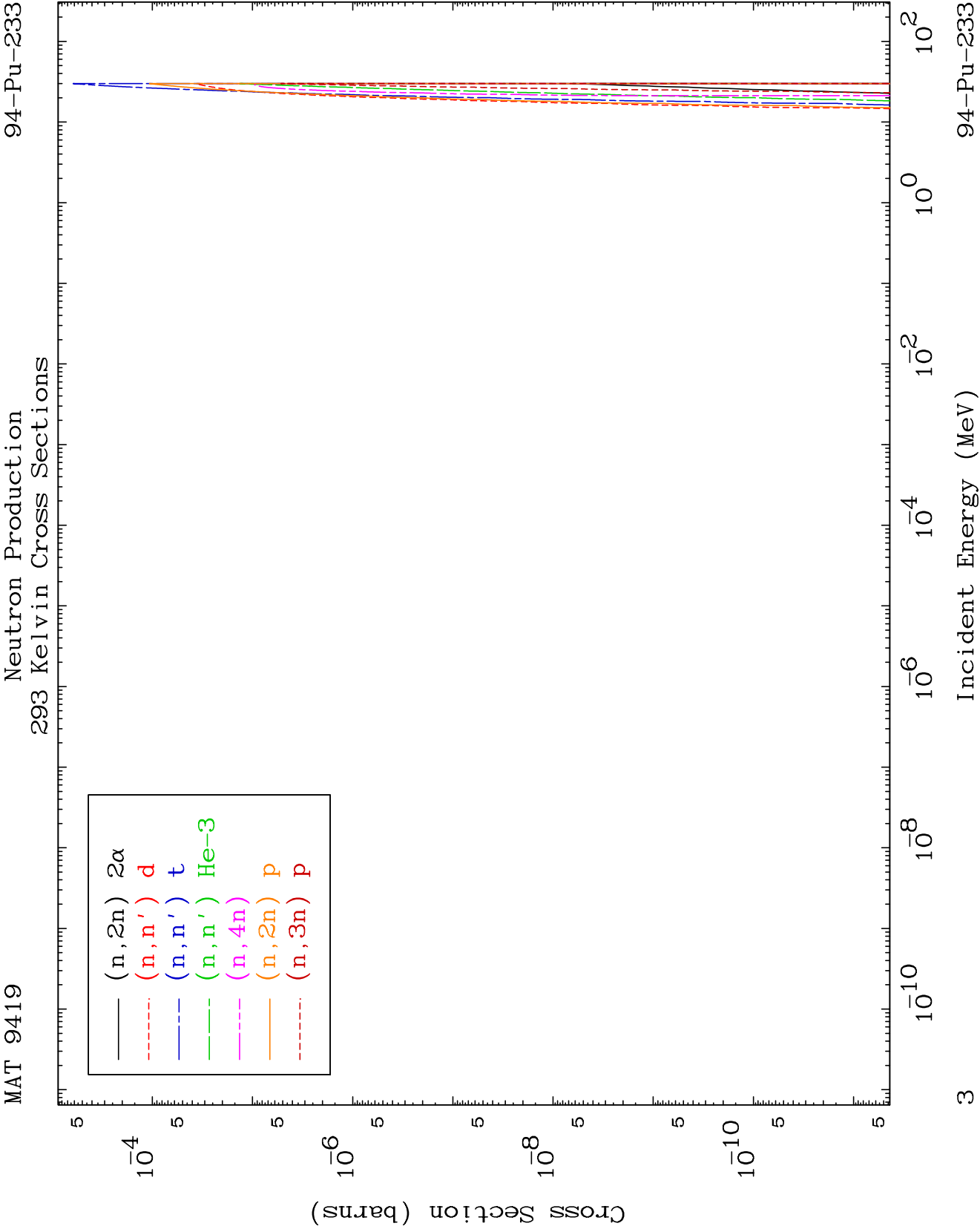


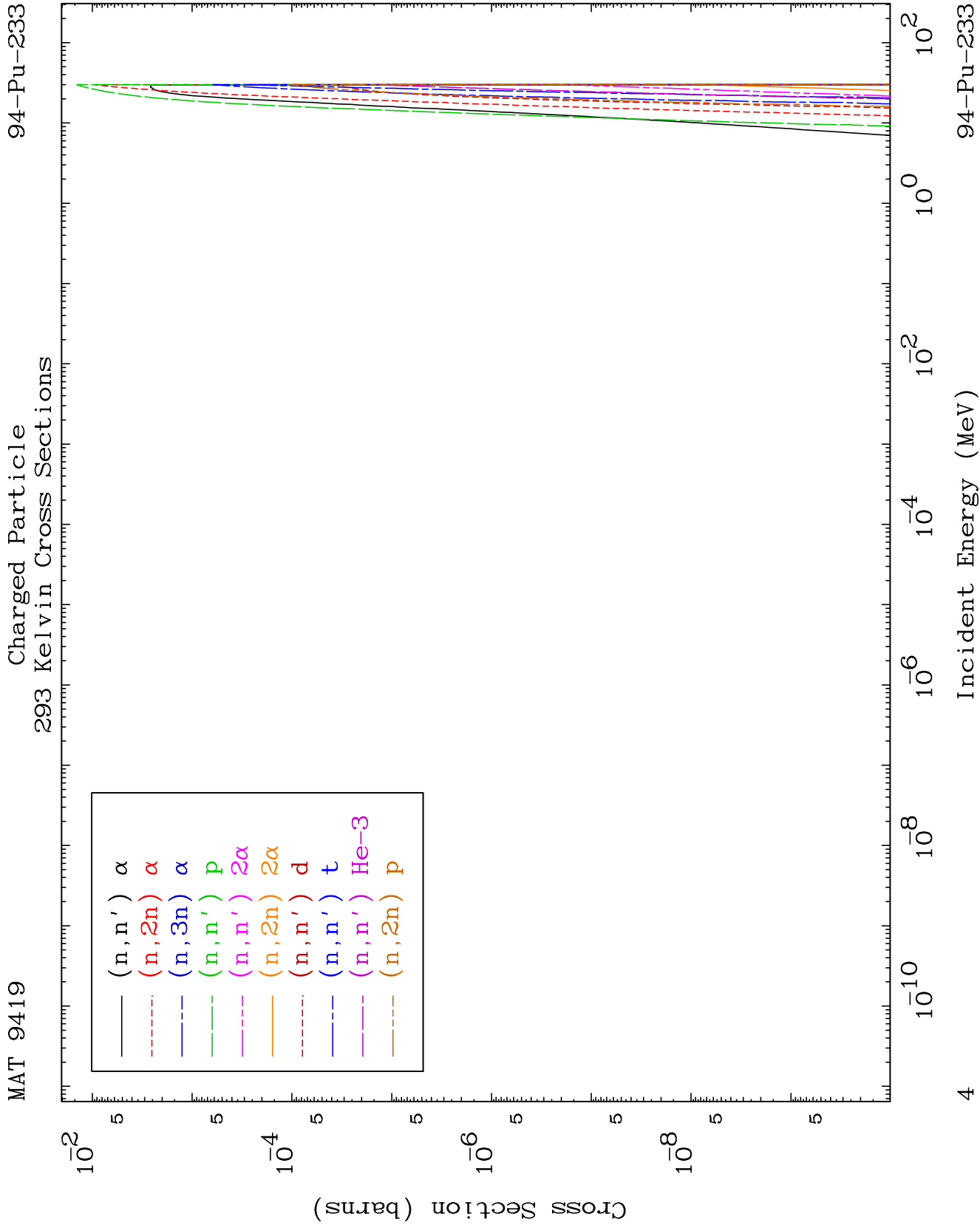
MAT 9419

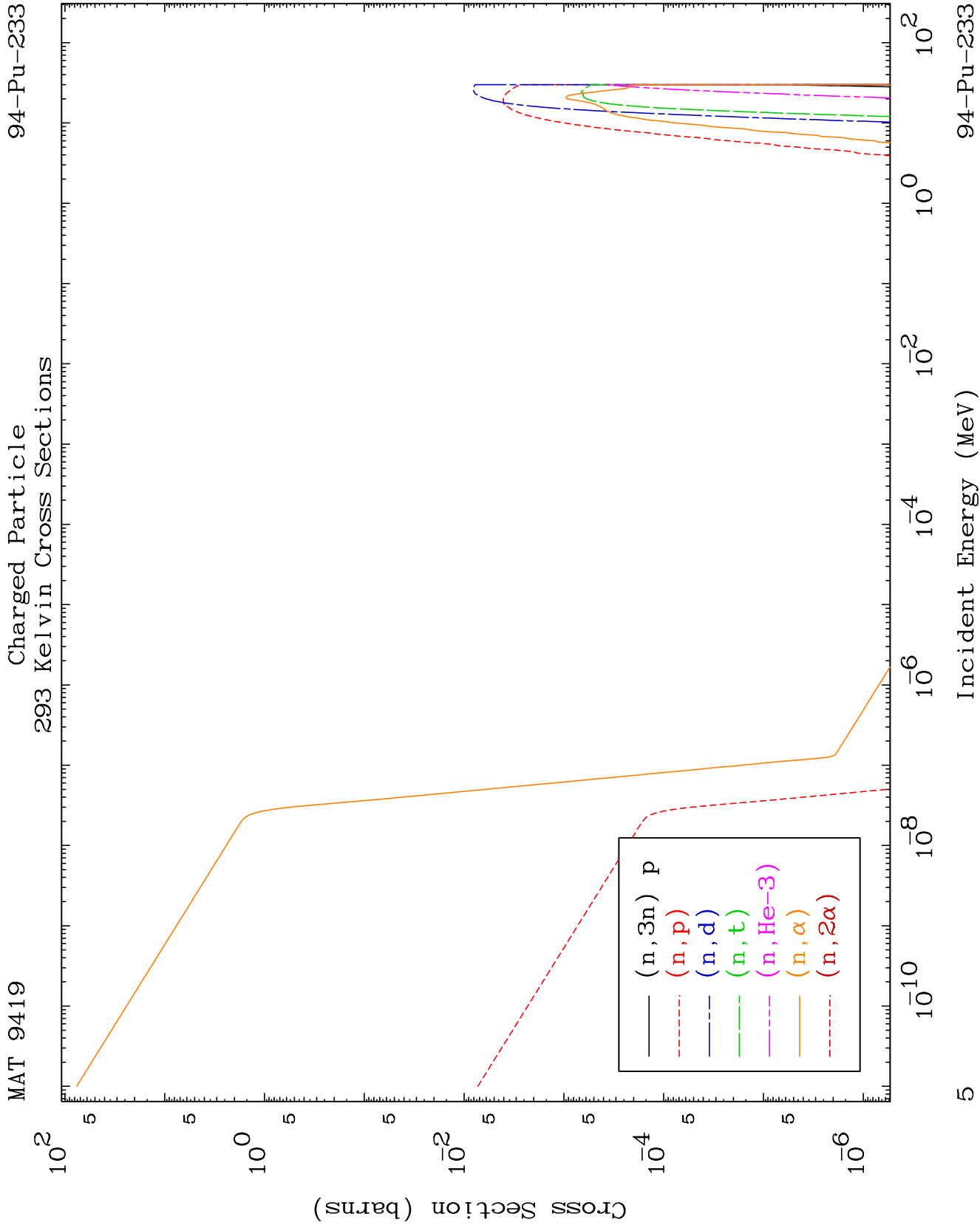
Neutron Production
293 Kelvin Cross Sections

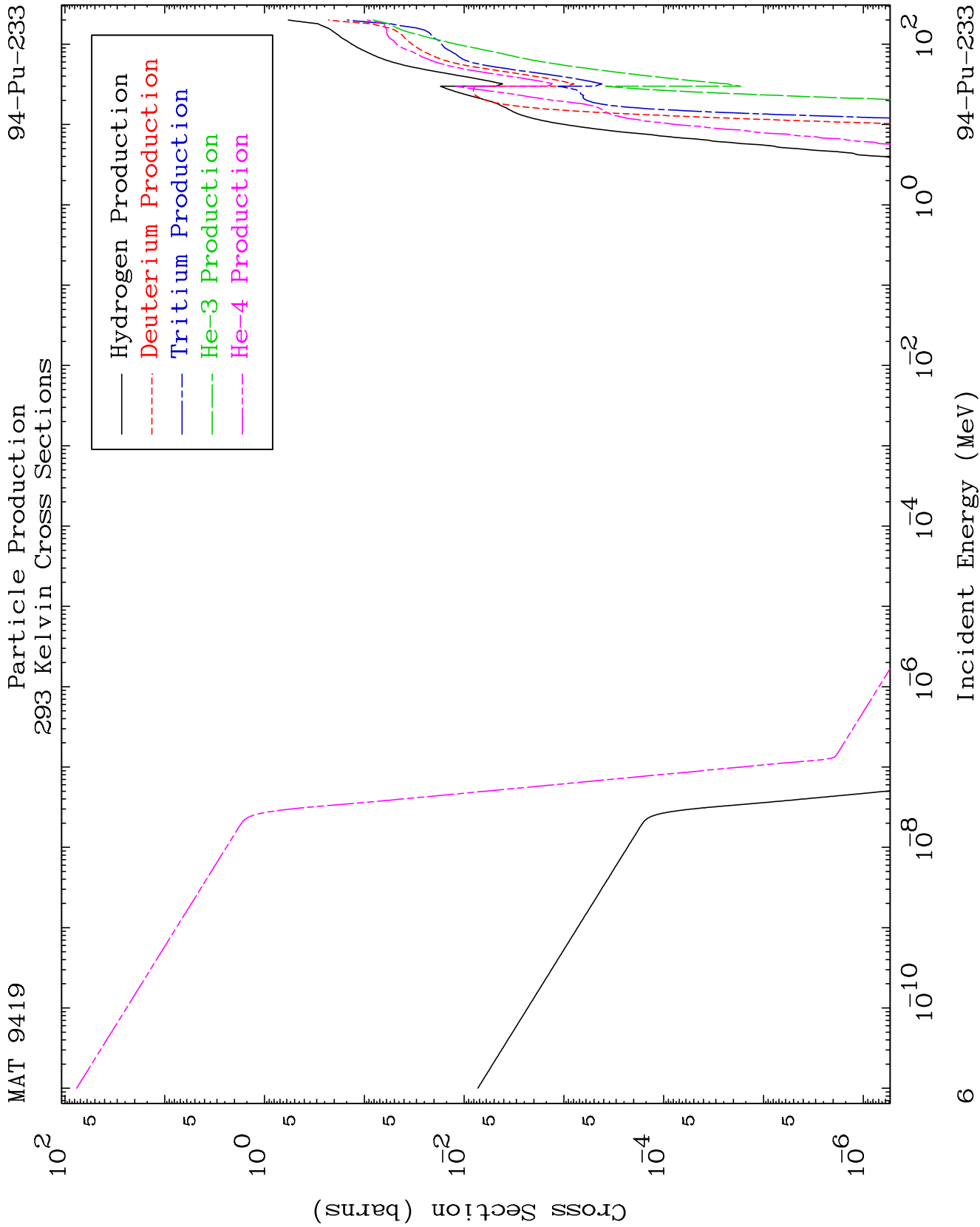
94-Pu-233







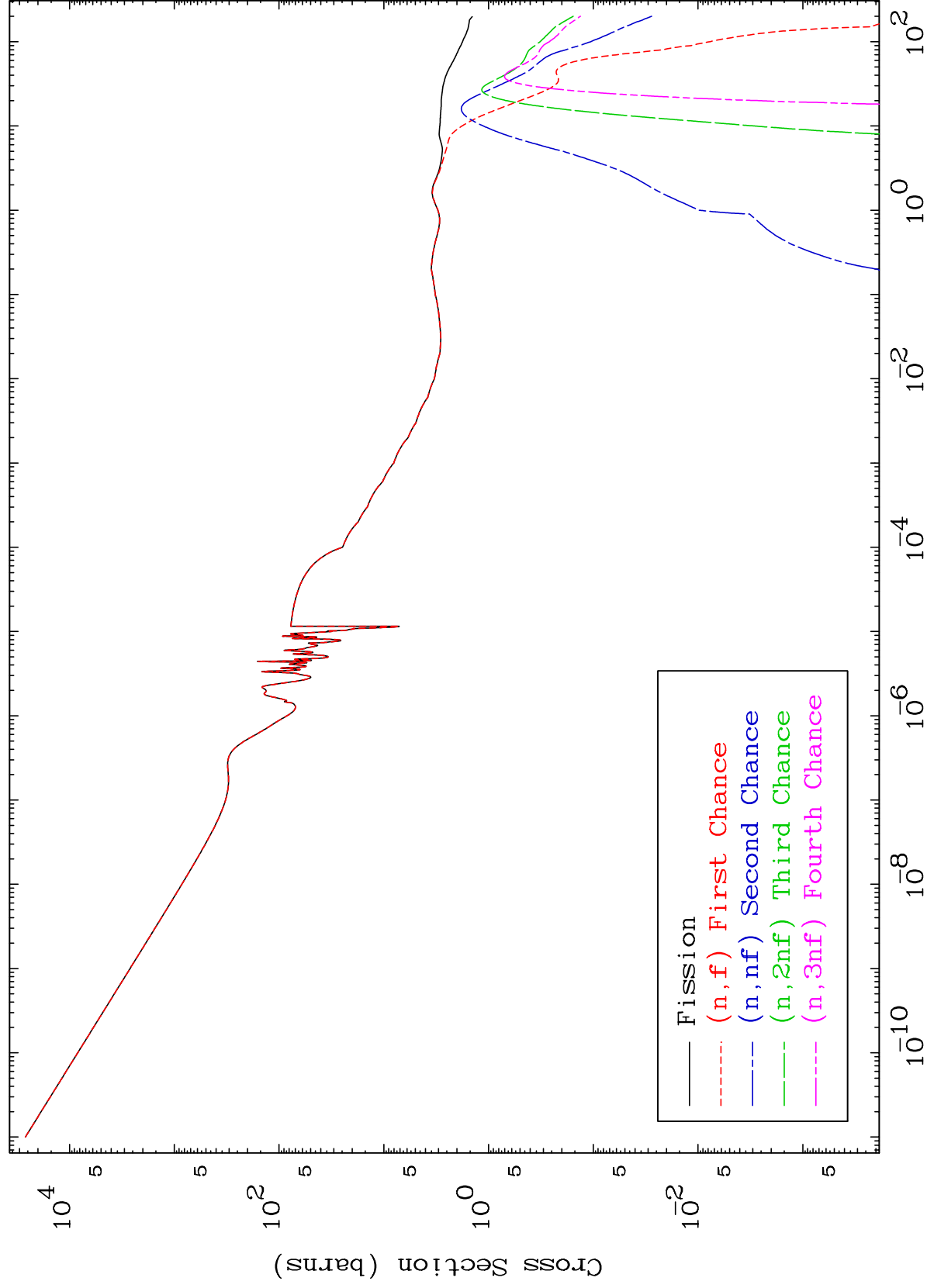




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Fission
293 Kelvin Cross Sections

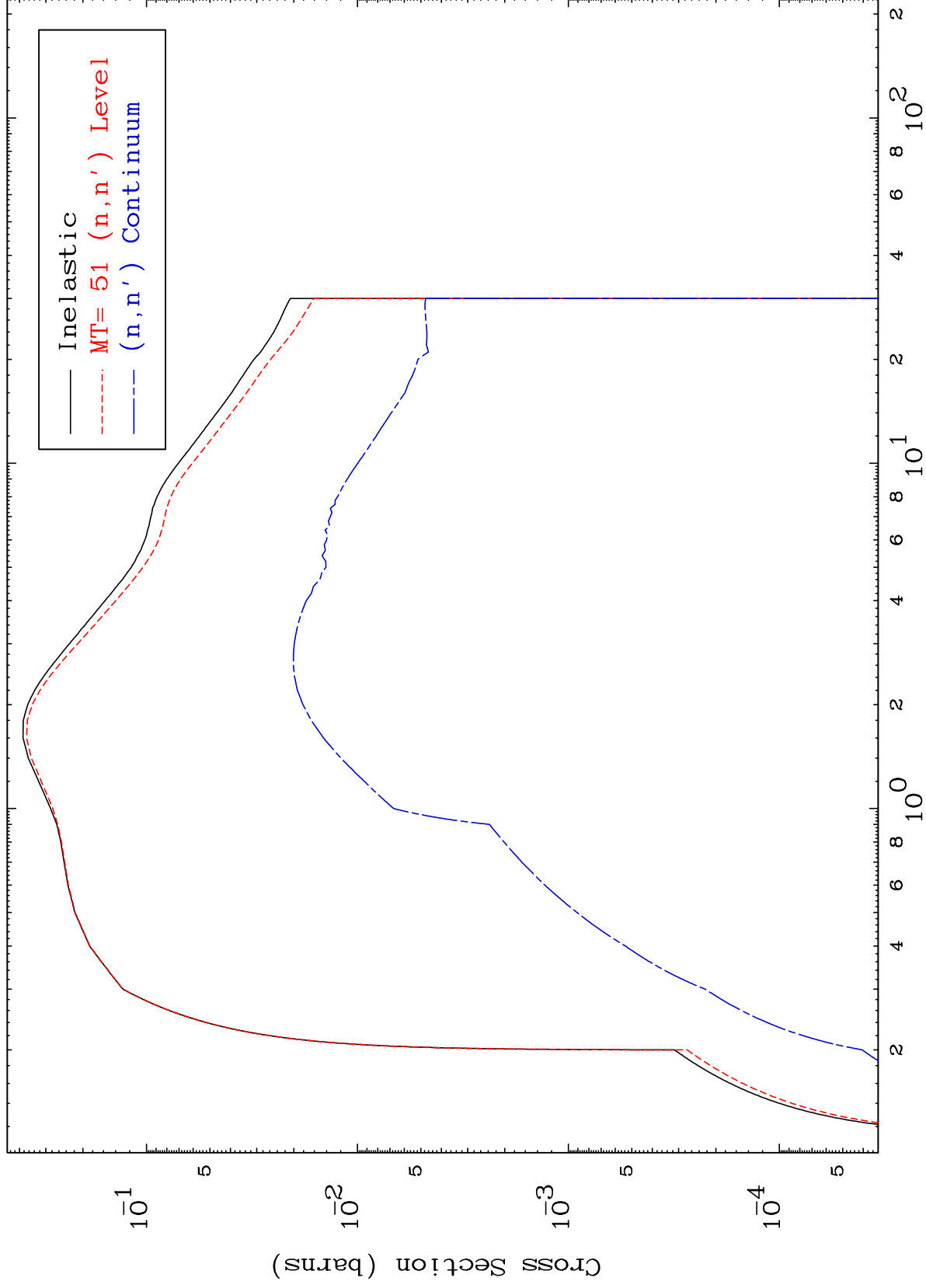
94-Pu-233



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94-Pu-233

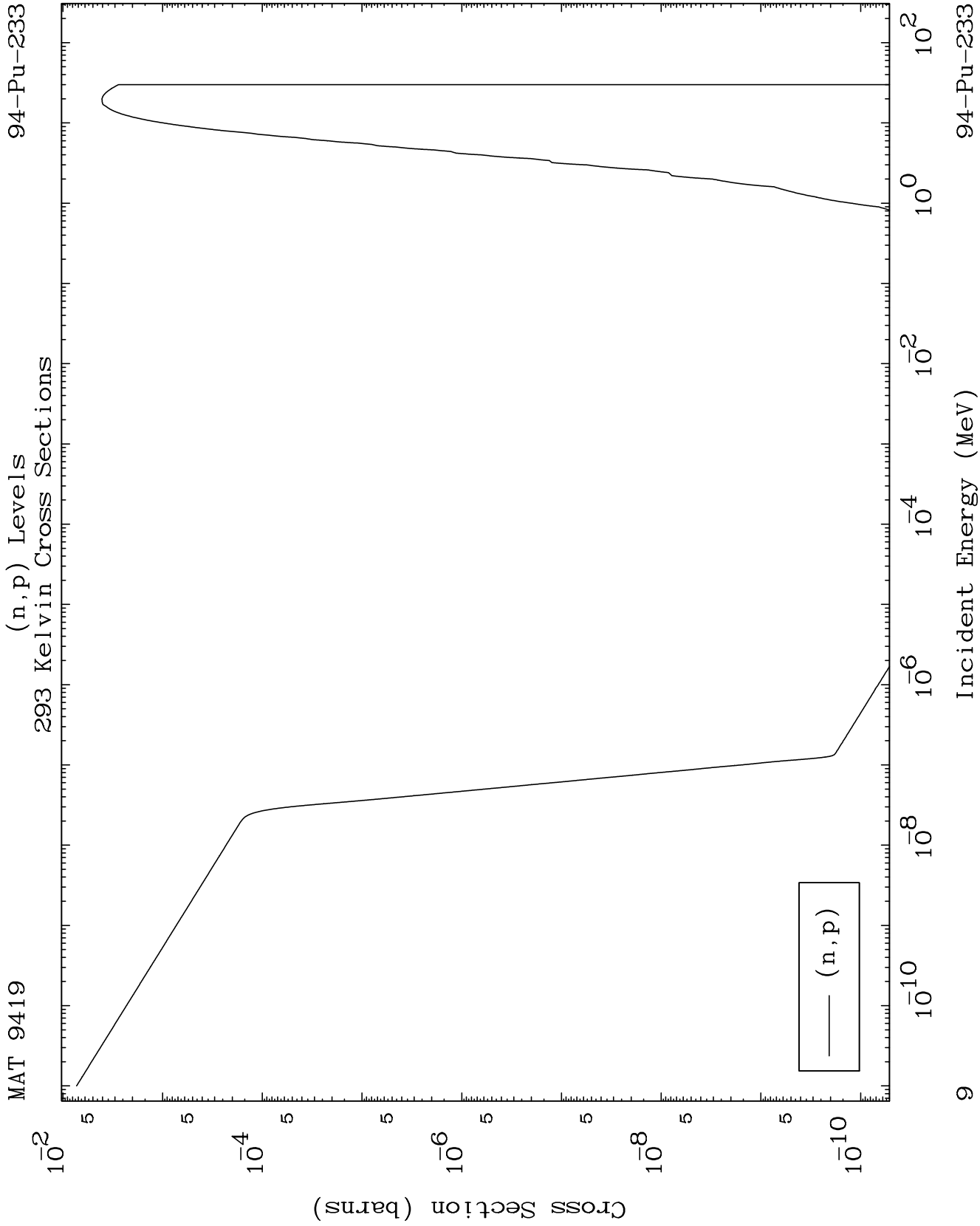
(n,n') Level
293 Kelvin Cross Sections

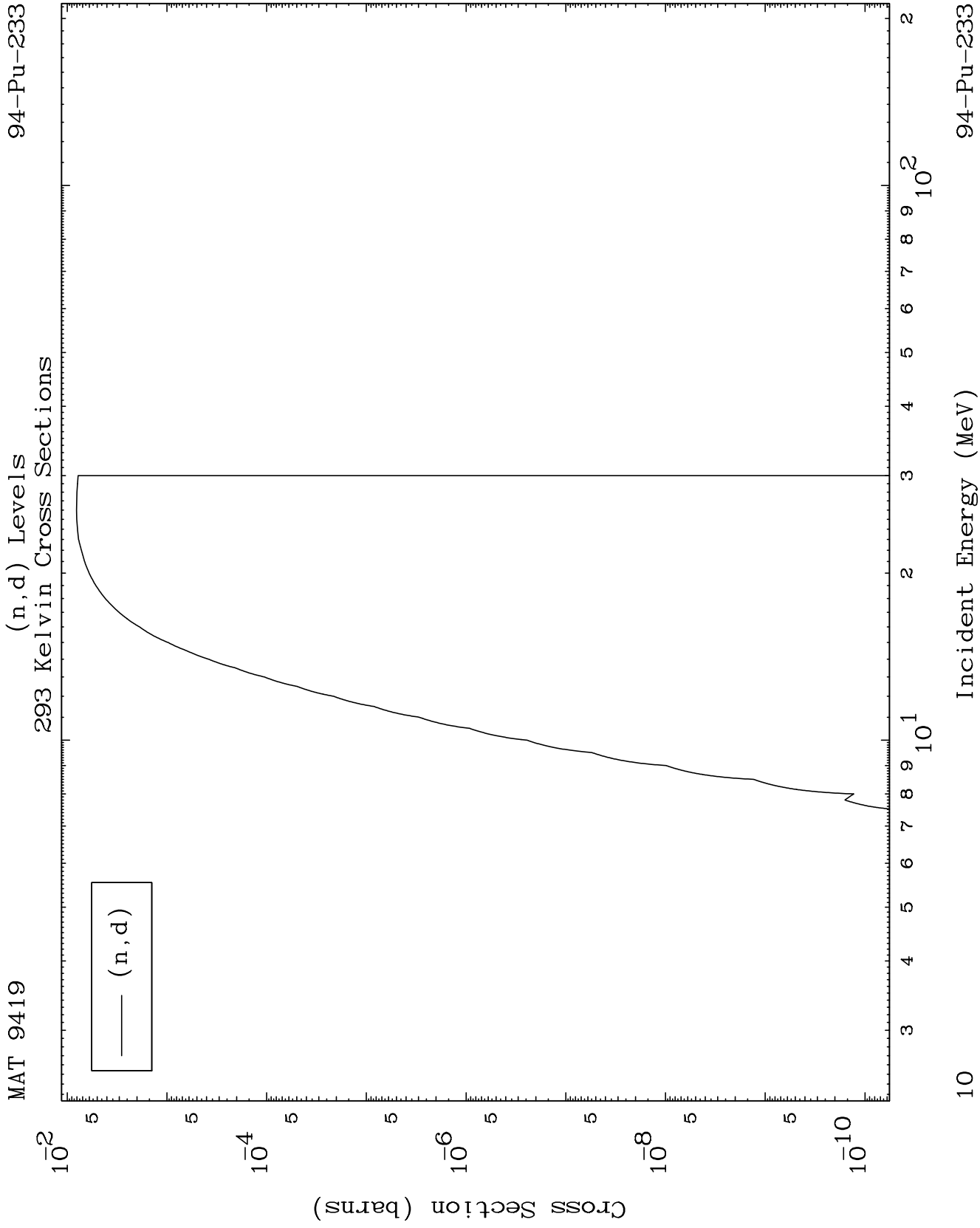


8

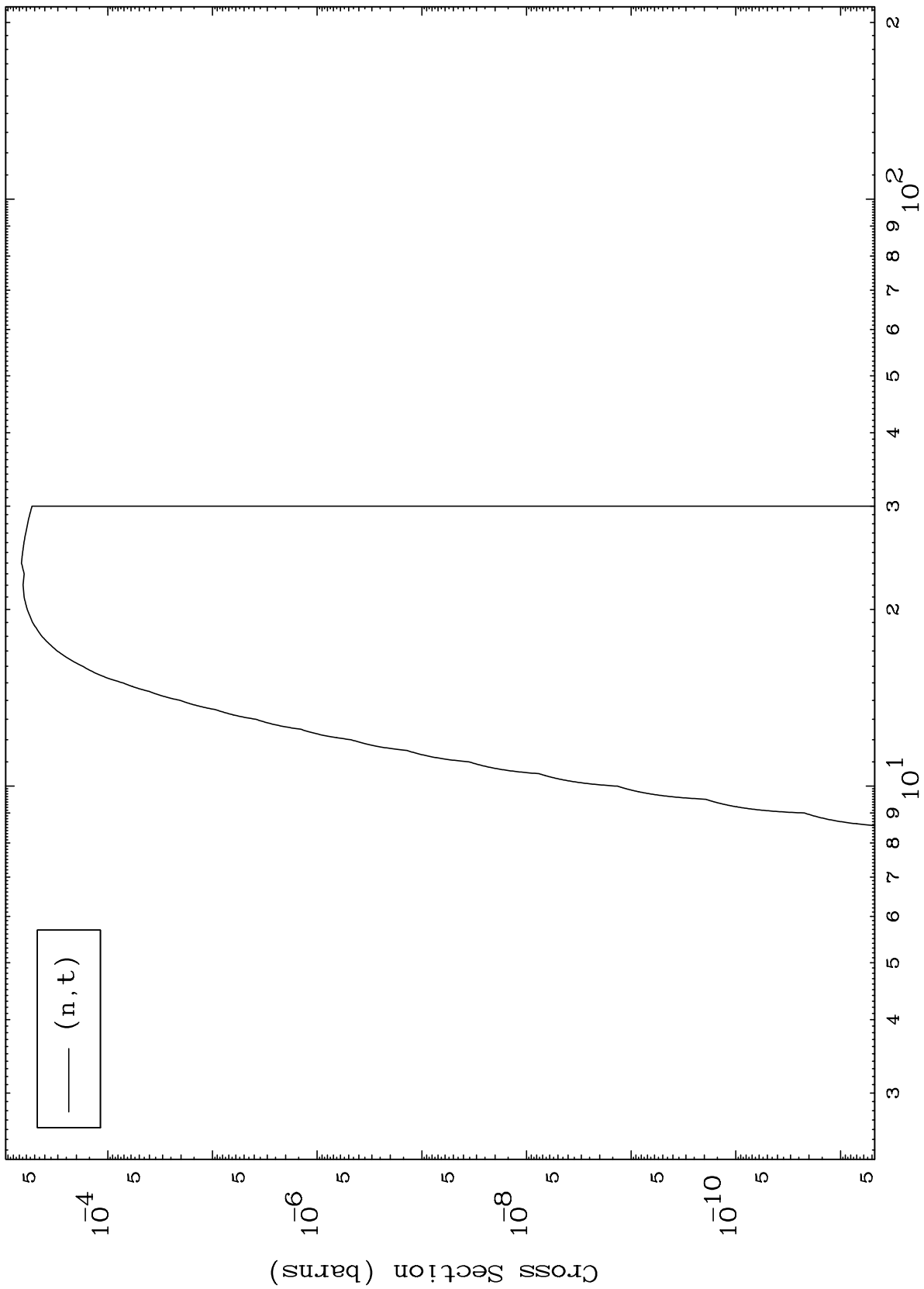
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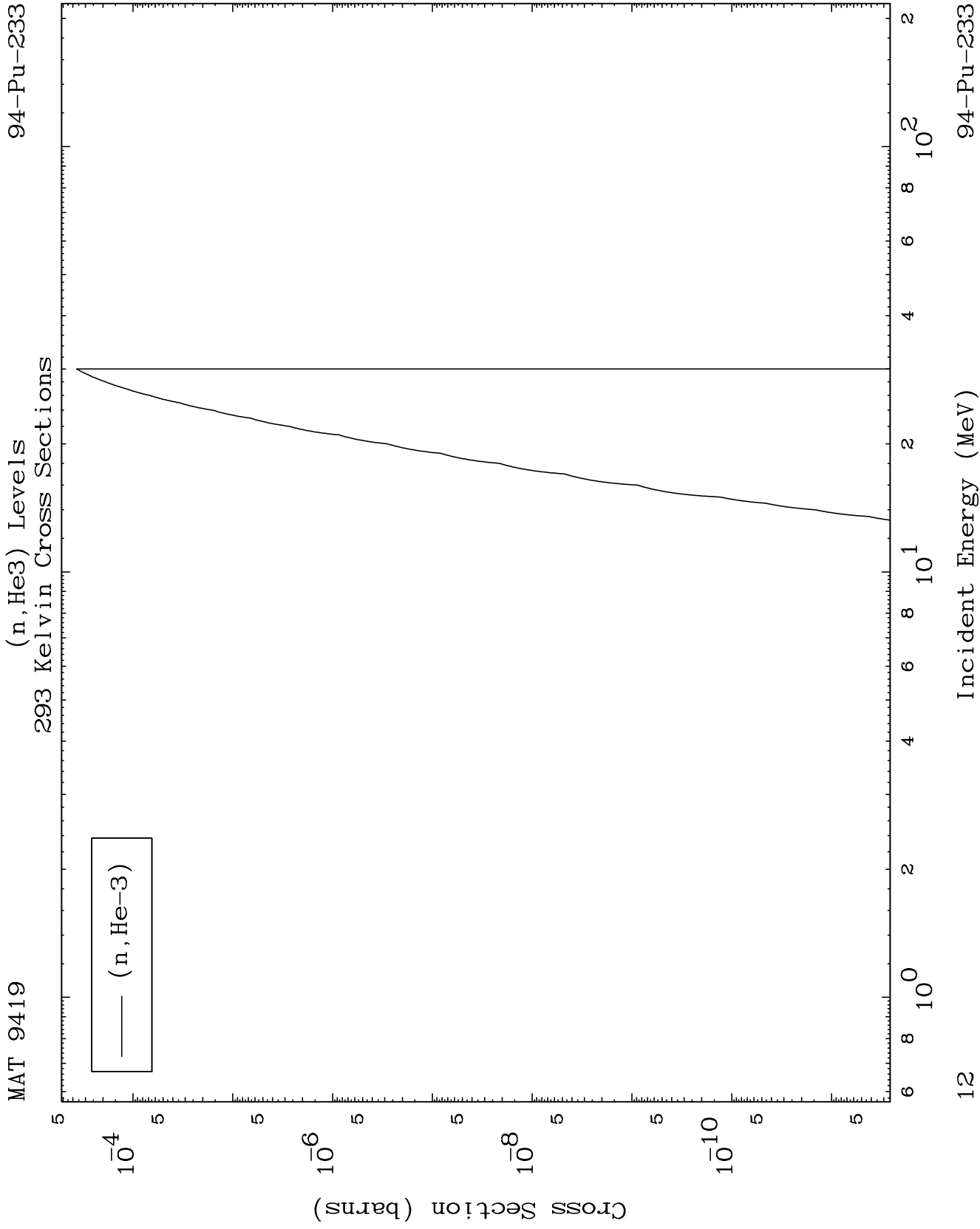
Incident Energy (MeV)

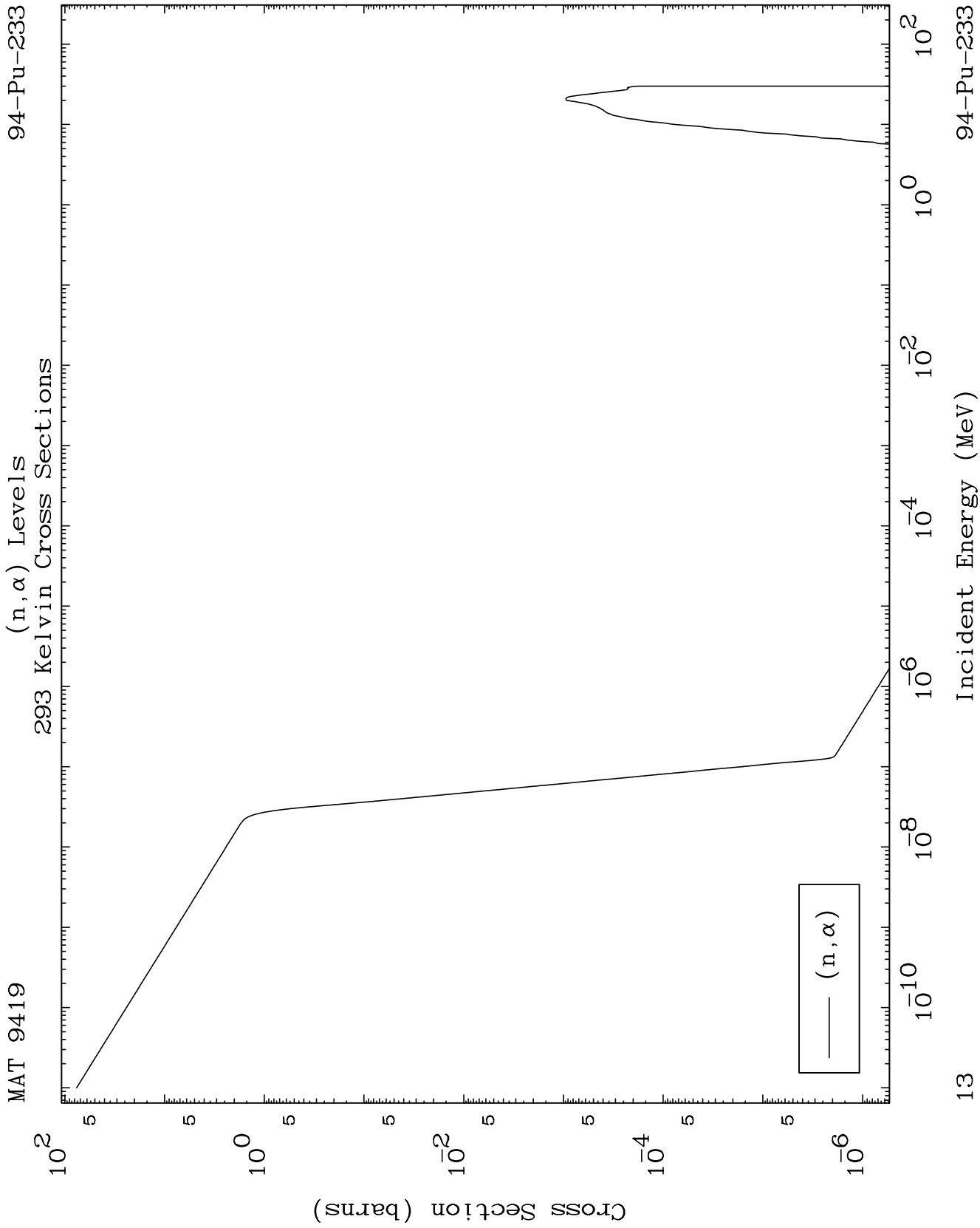


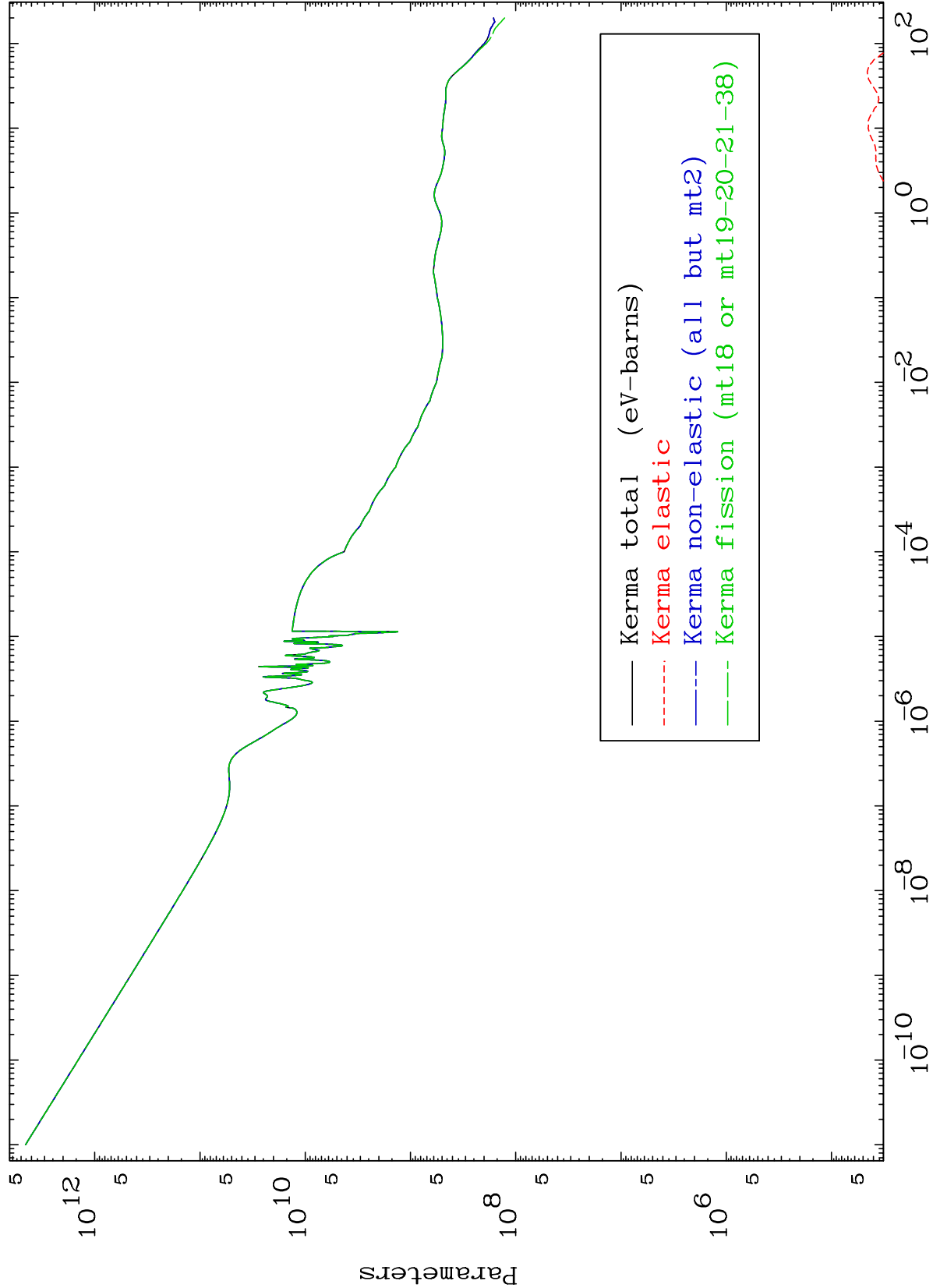


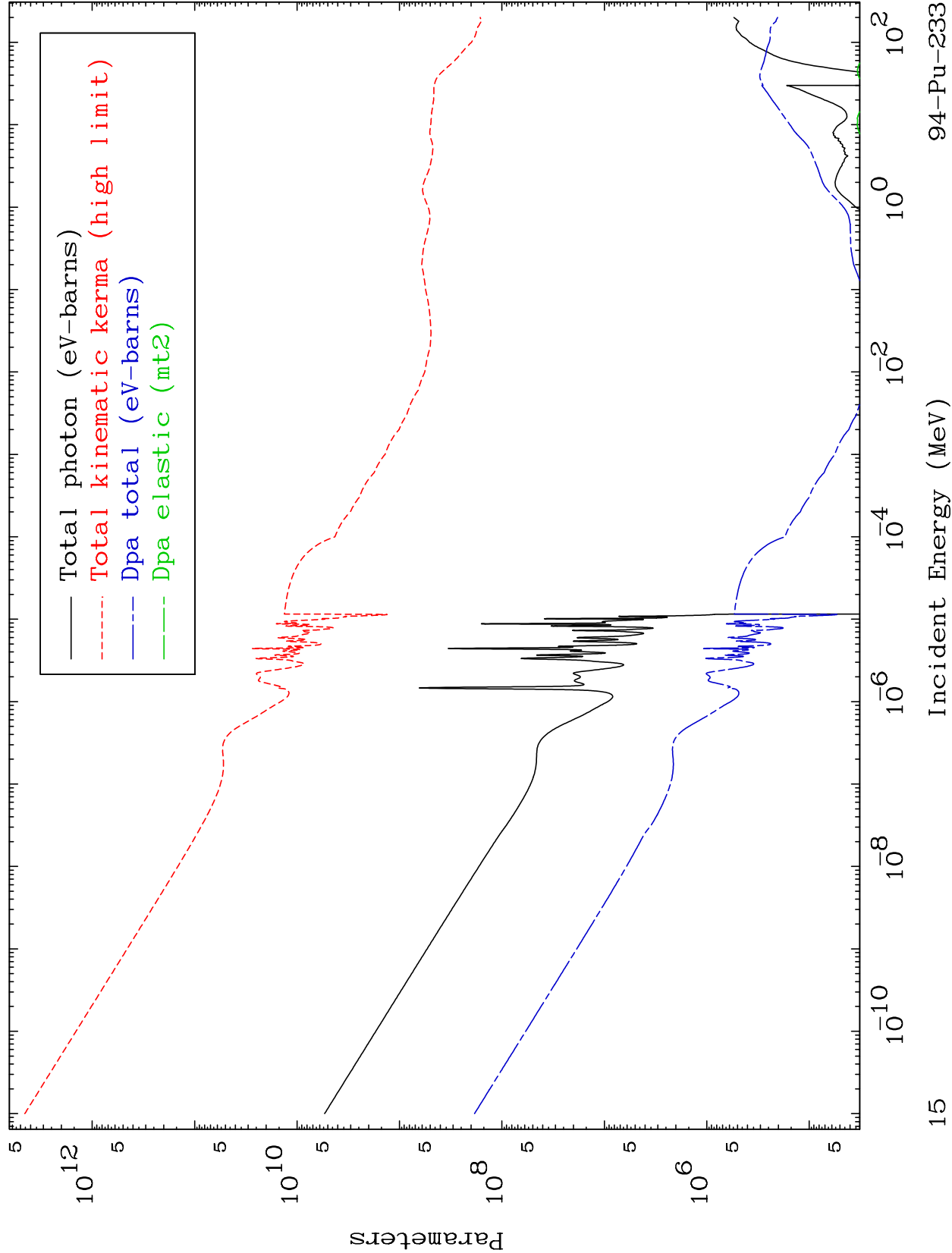
(n,t) Levels
293 Kelvin Cross Sections

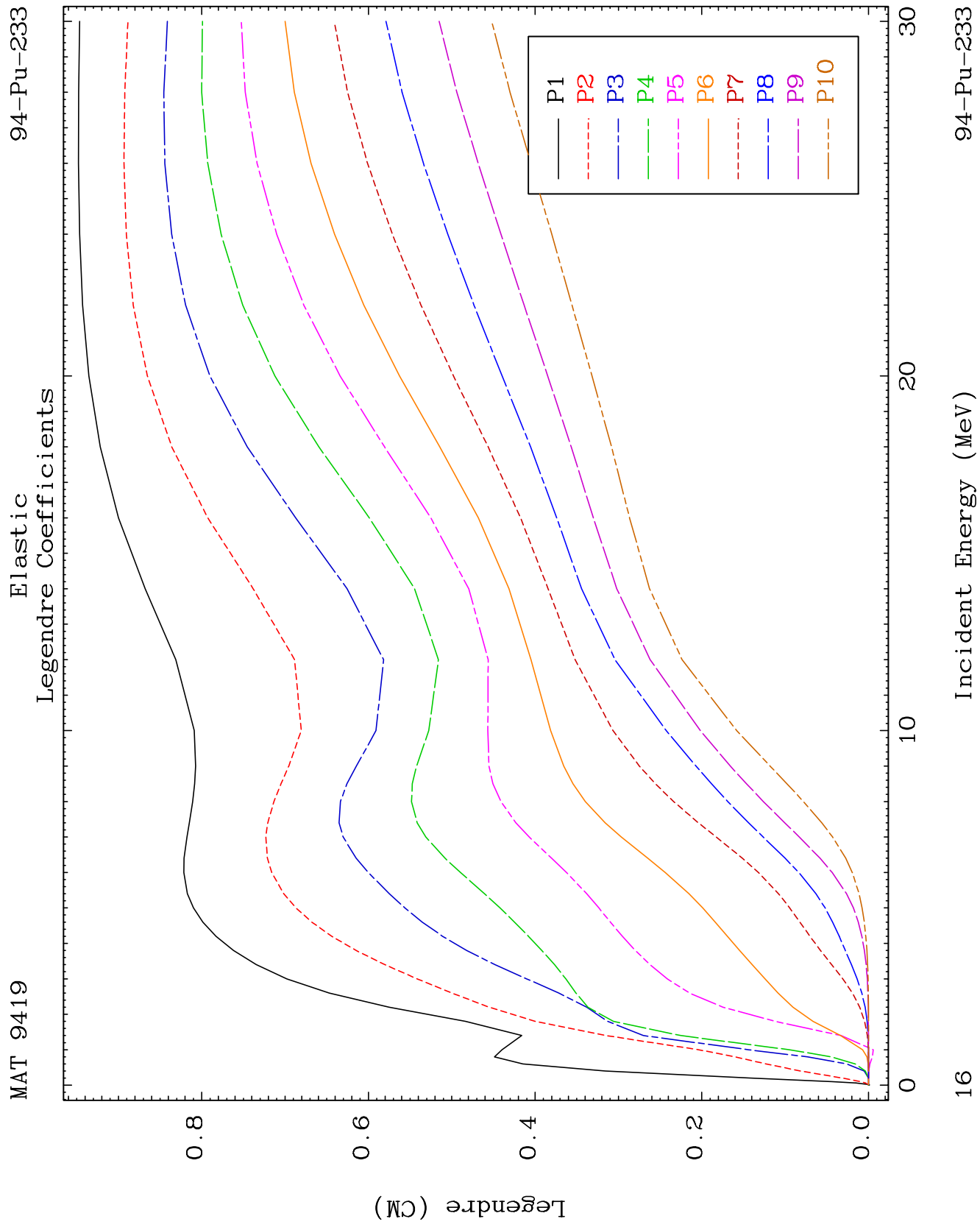


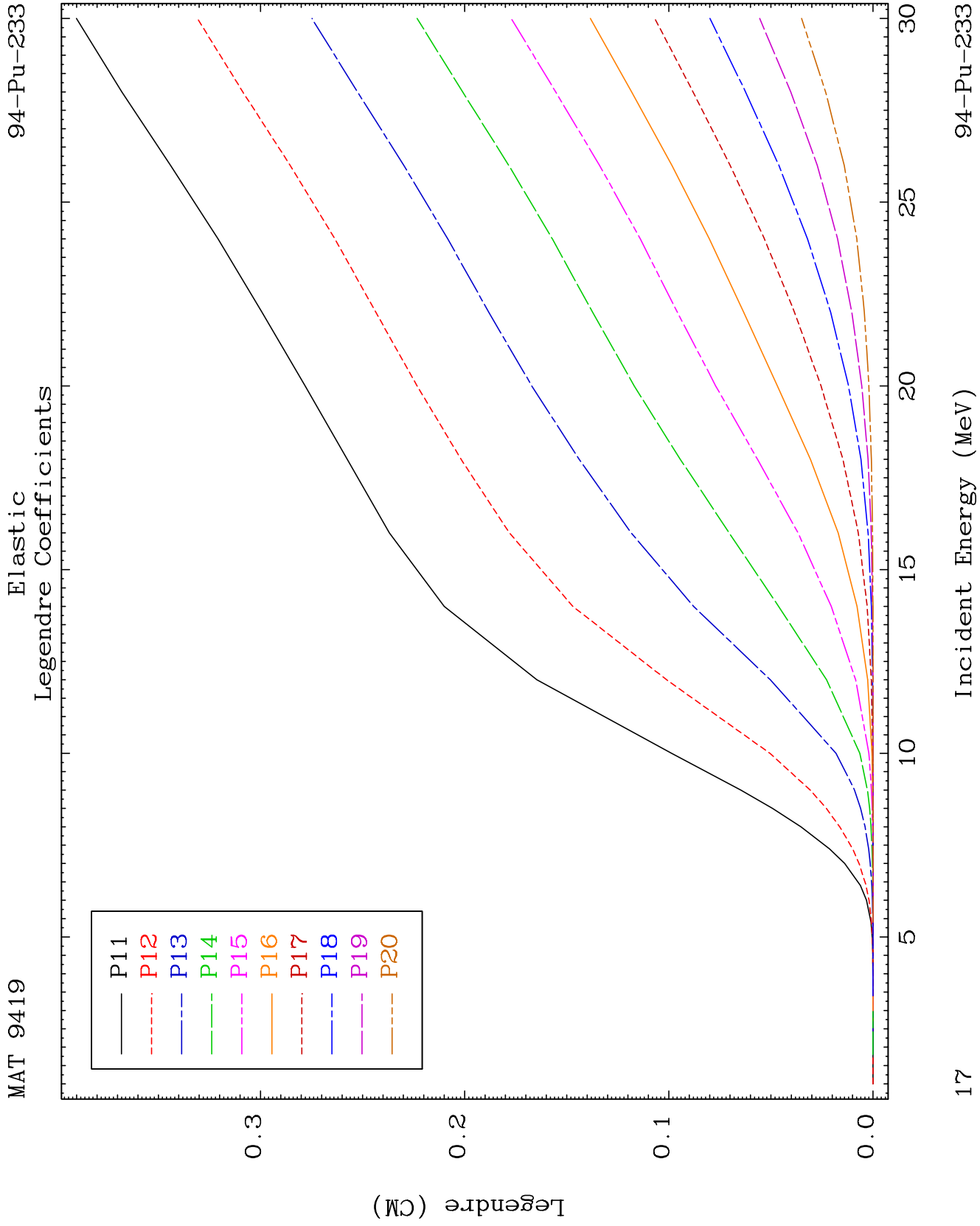


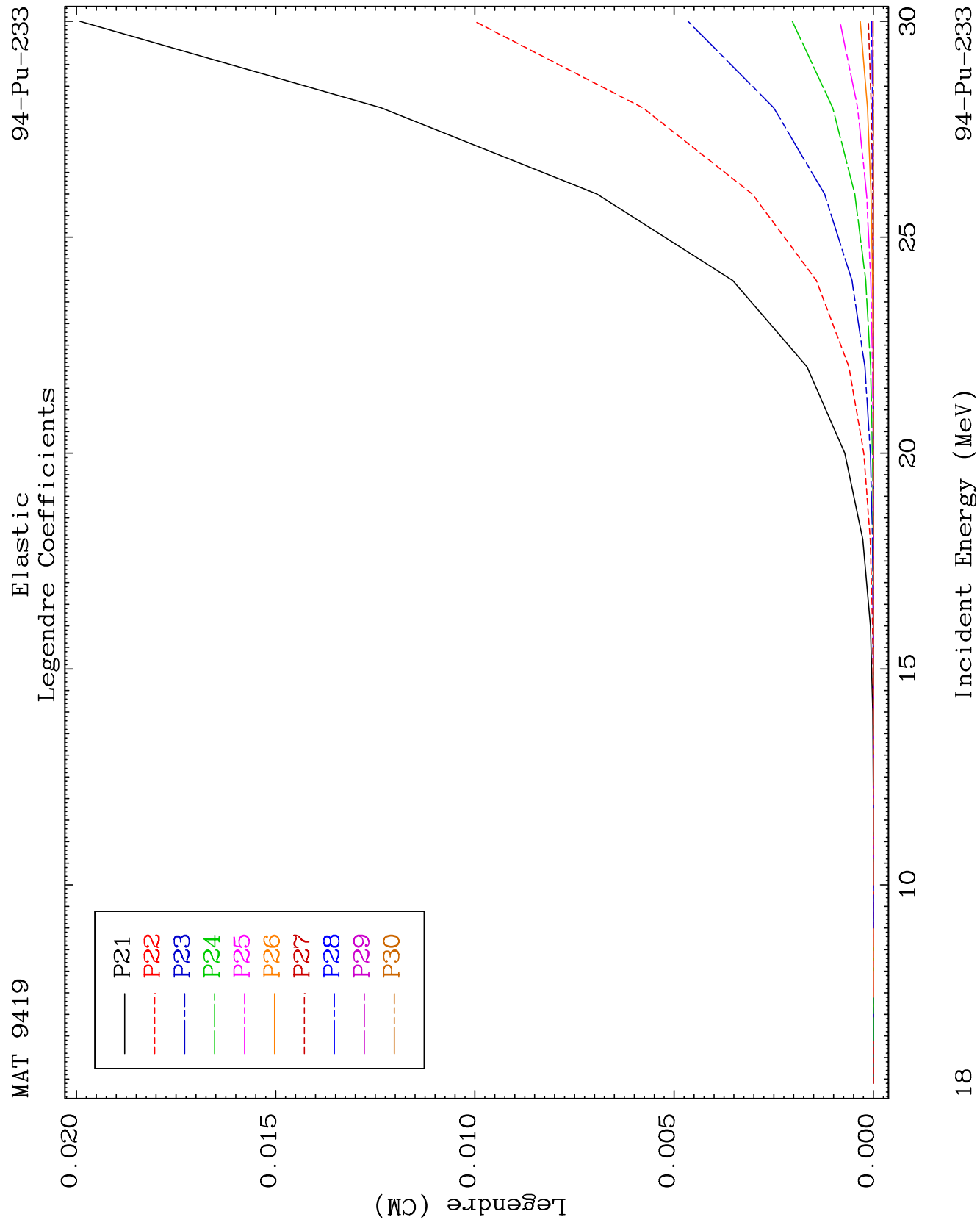


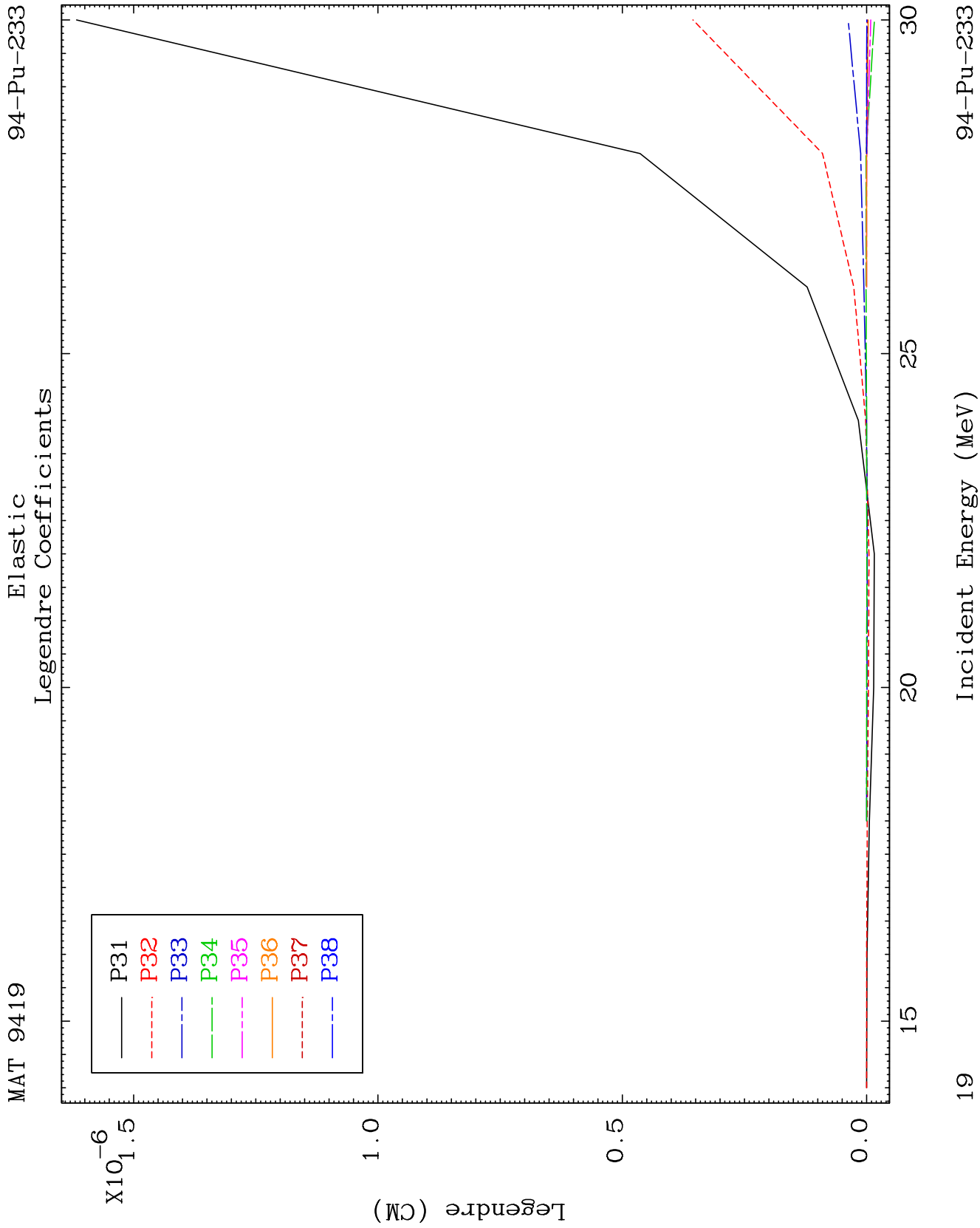








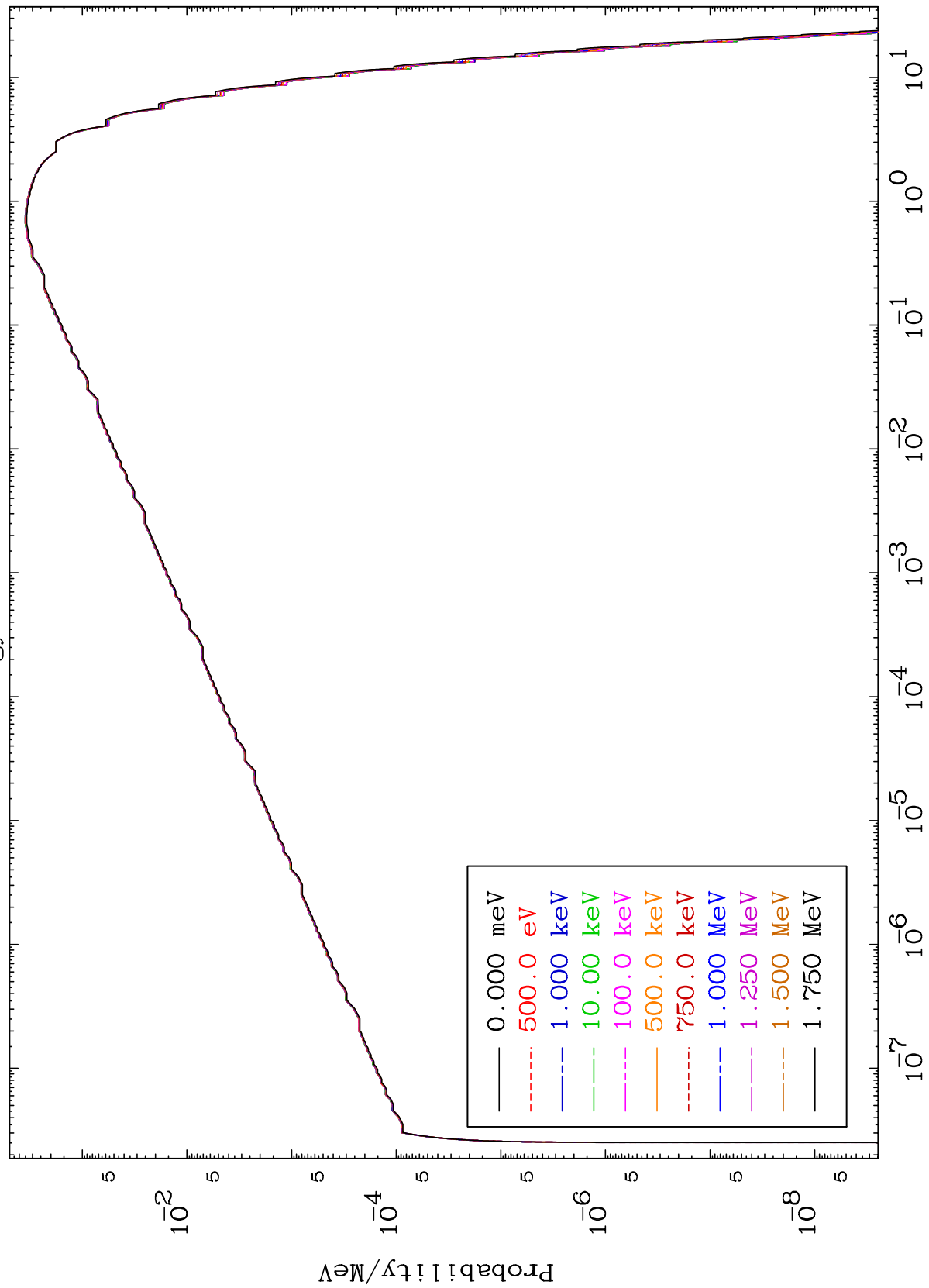


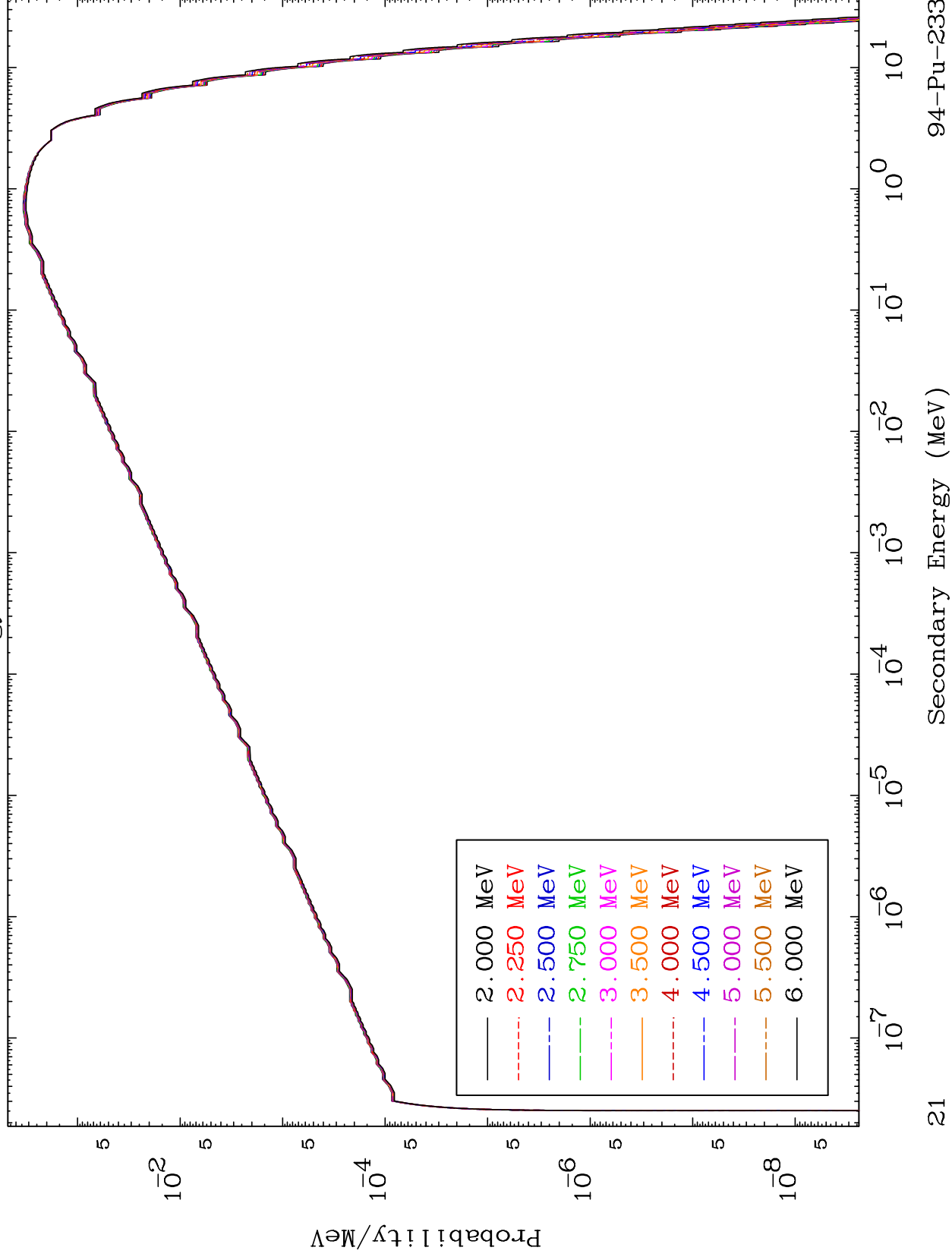


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Fission
Energy Distribution

94-Pu-233

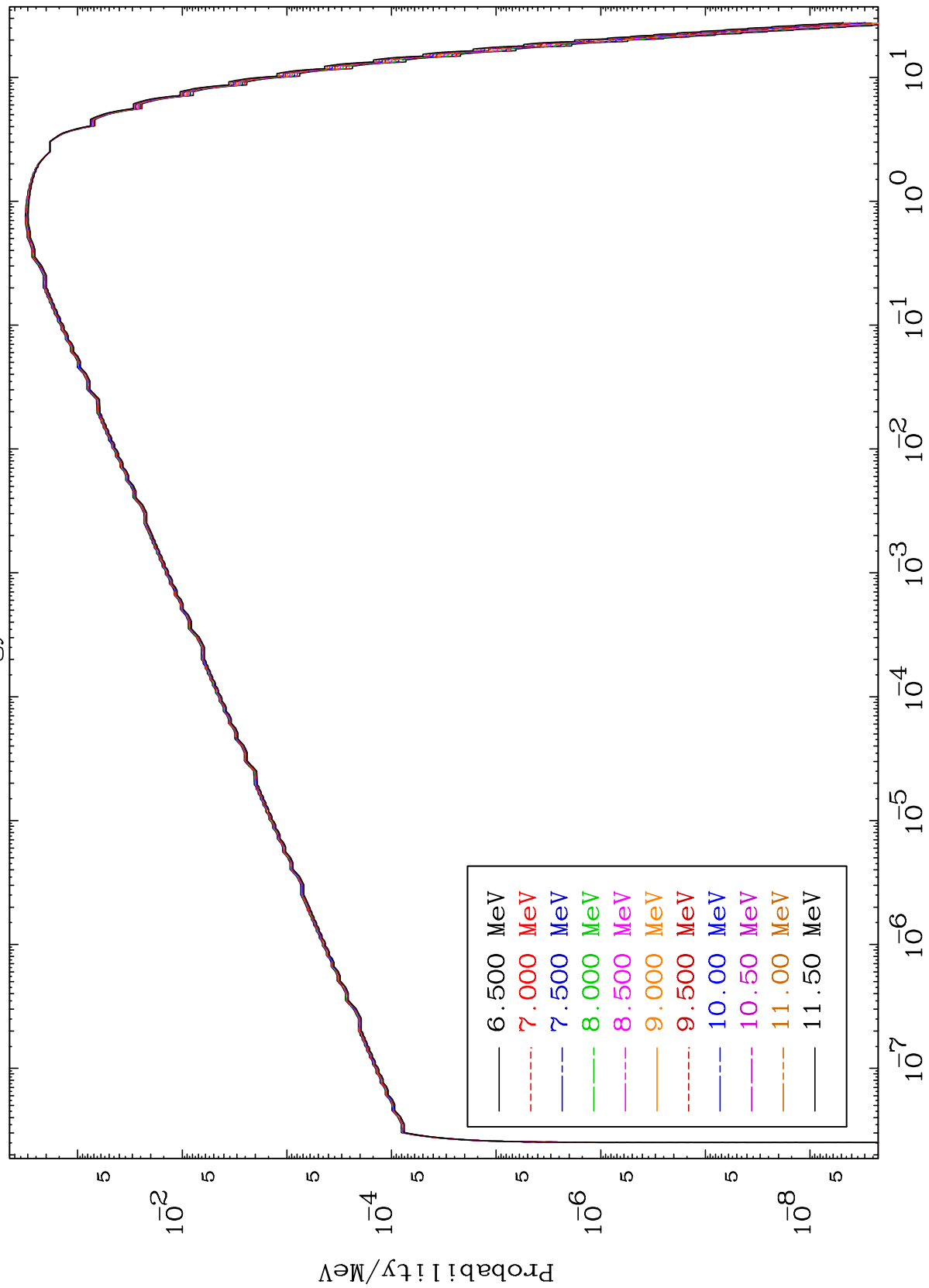


Fission
Energy Distribution

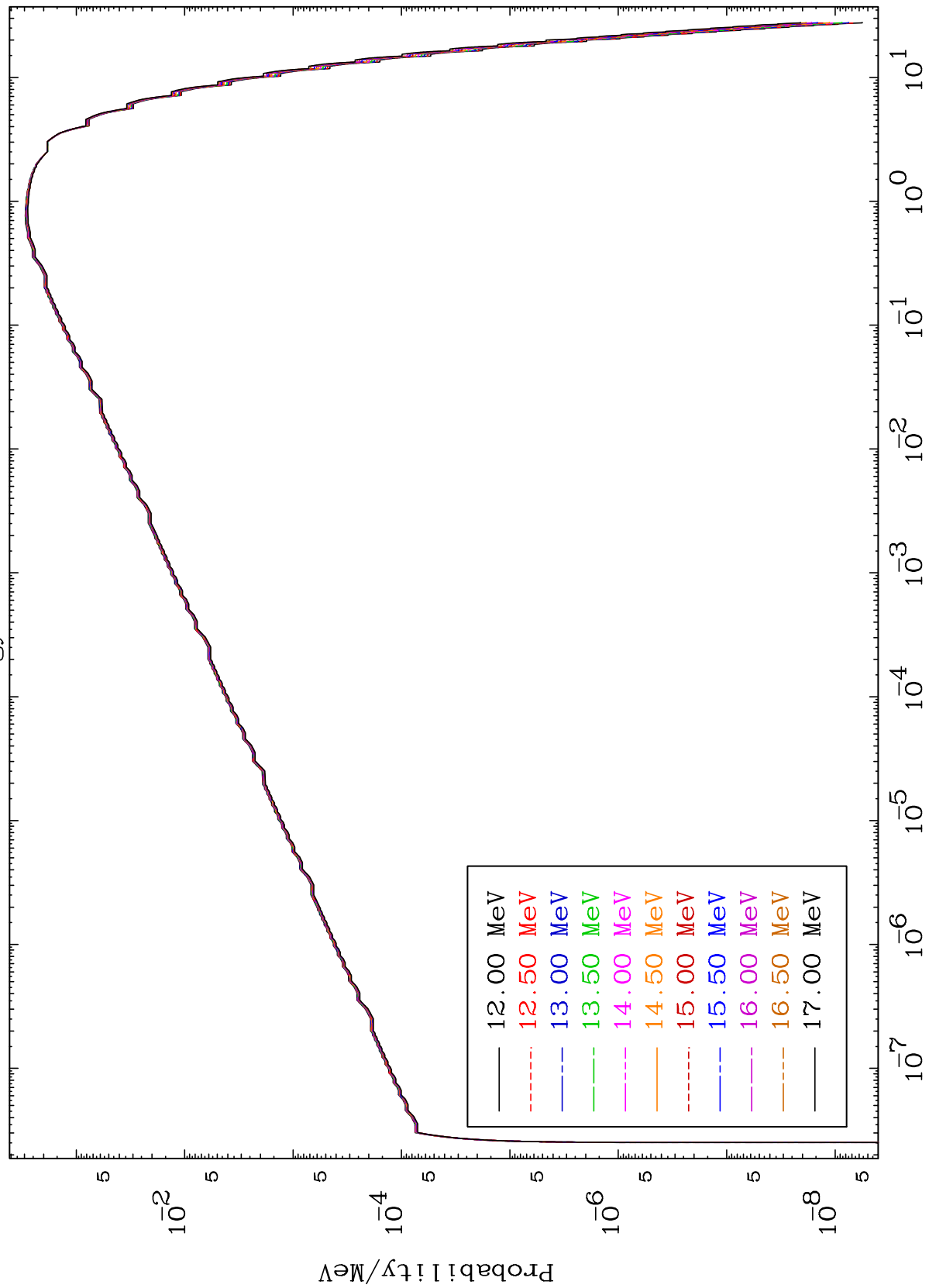
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Fission
Energy Distribution

94-Pu-233



Fission Energy Distribution



Fission Energy Distribution

