

Program EVALPLOT
(Version 2017-1)

by

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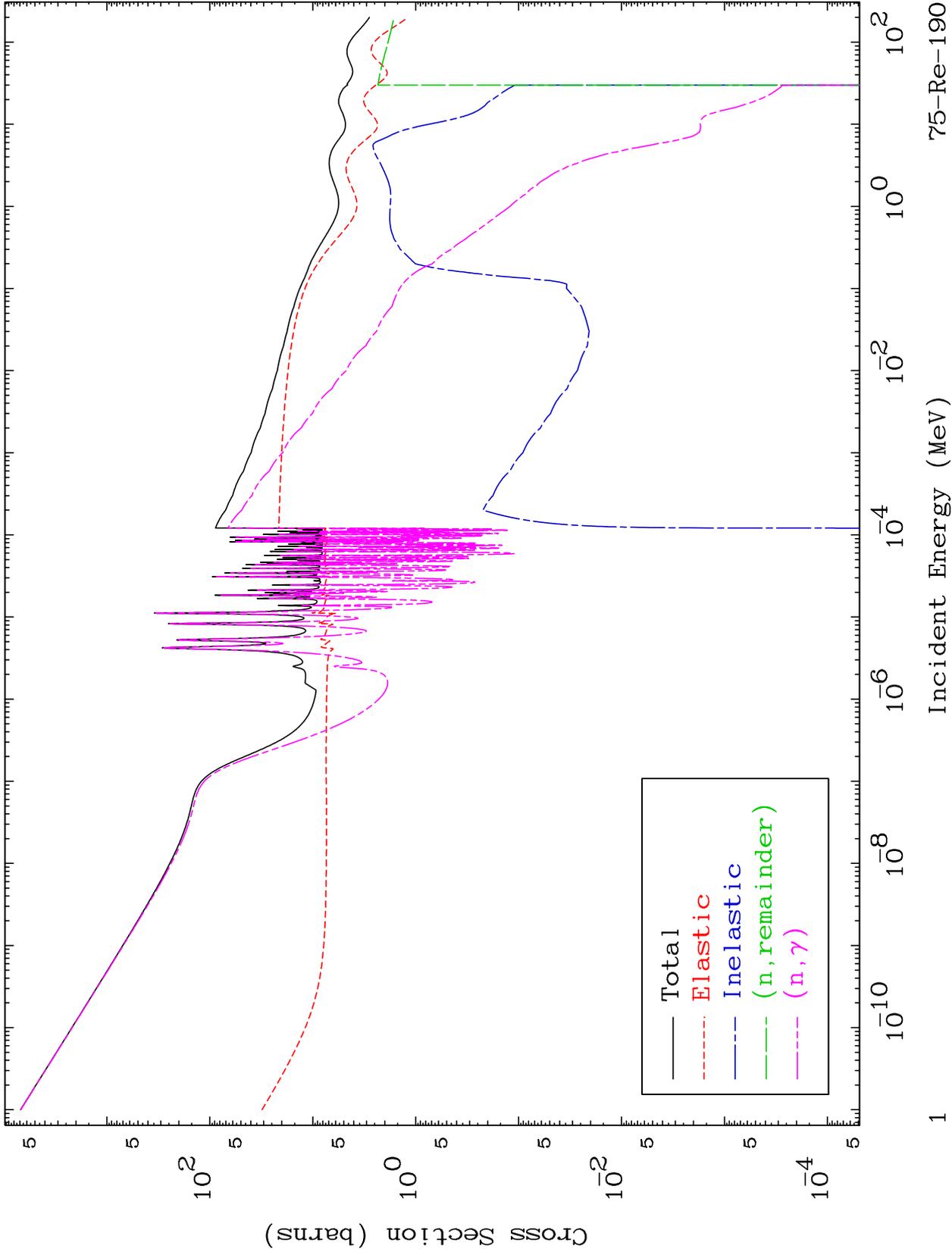
Press Mouse Button to Start

MAT 7541

Major

293 Kelvin Cross Sections

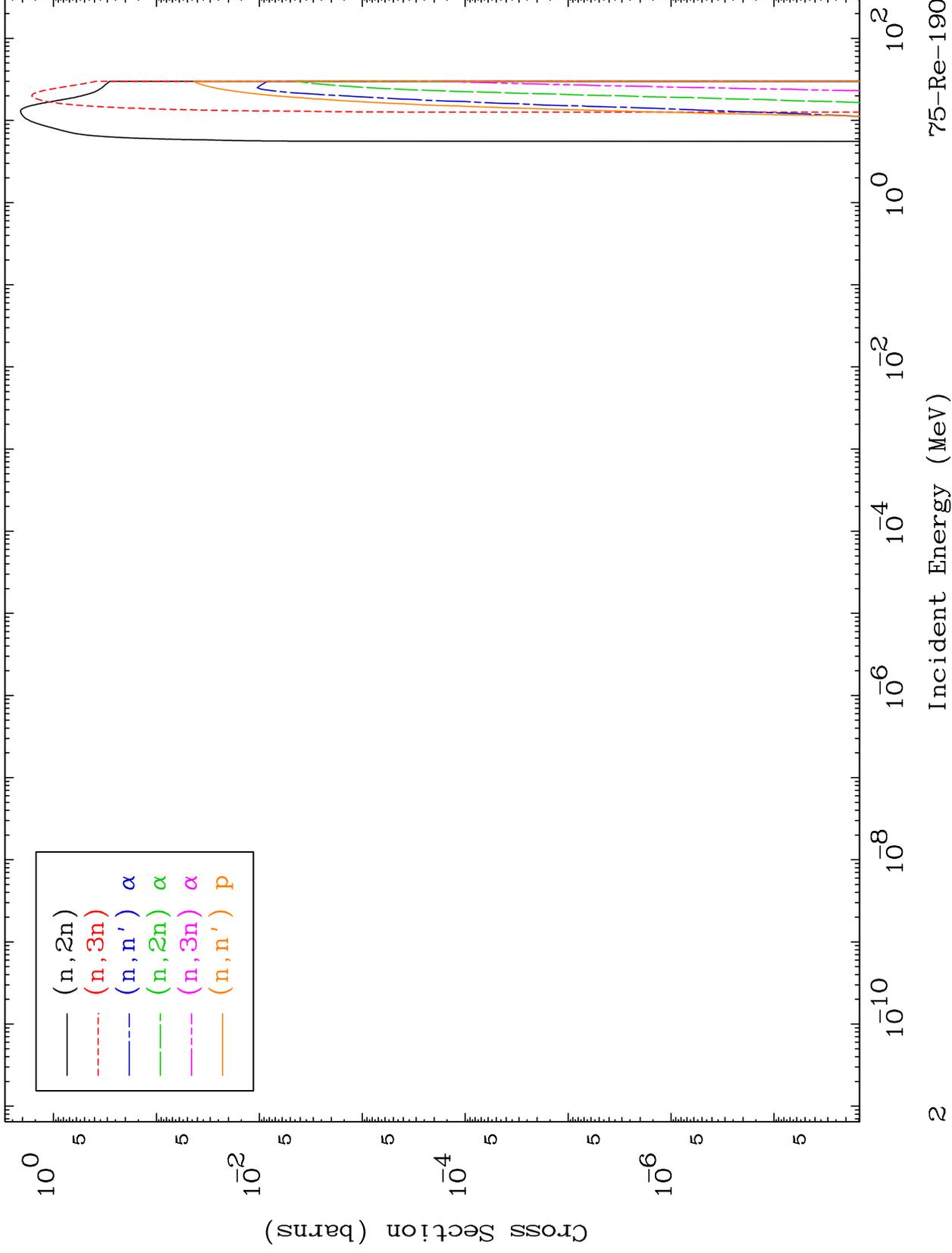
75-Re-190



MAT 7541

Neutron Production
293 Kelvin Cross Sections

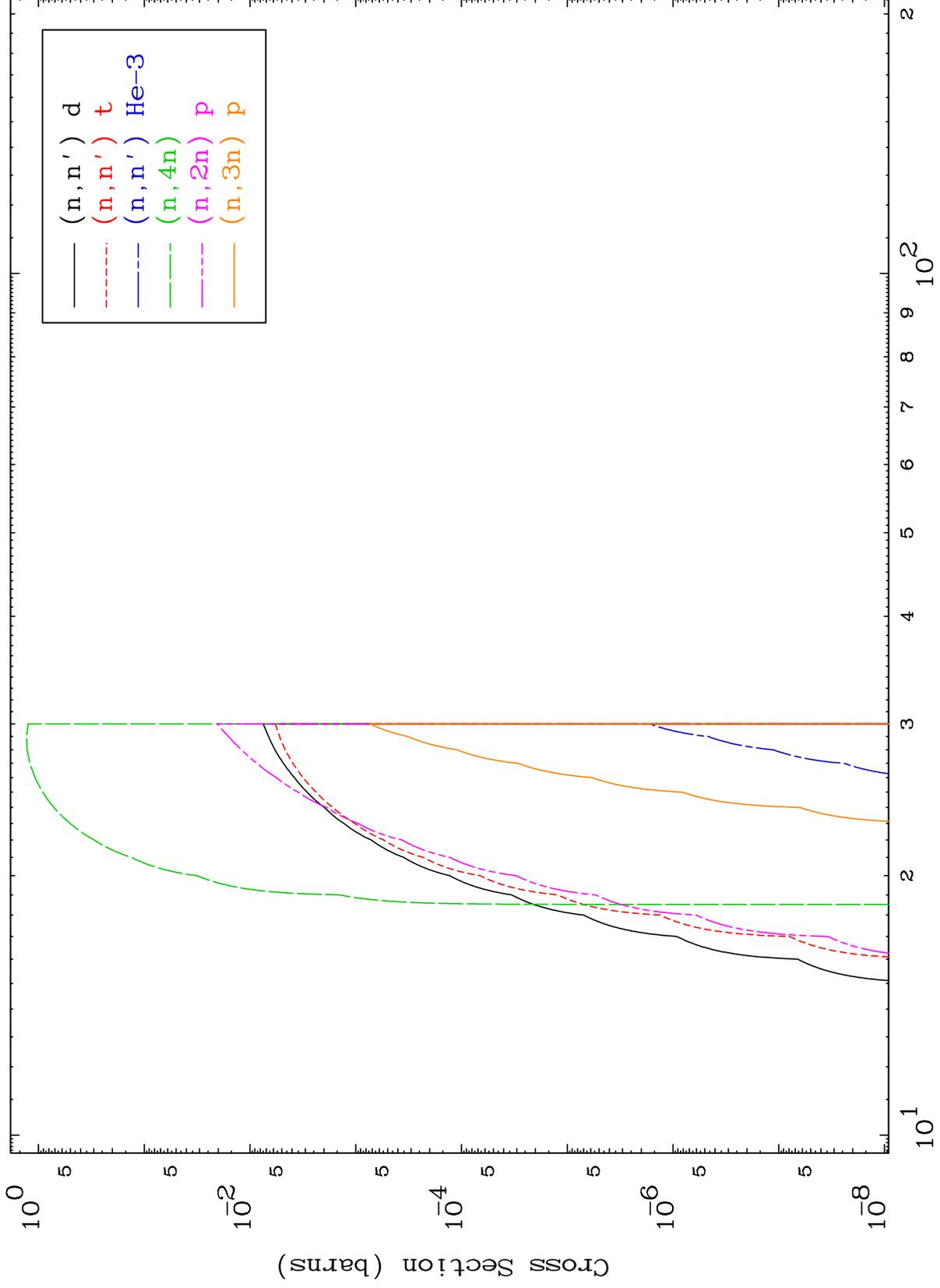
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MAT 7541

Neutron Production
293 Kelvin Cross Sections

75-Re-190



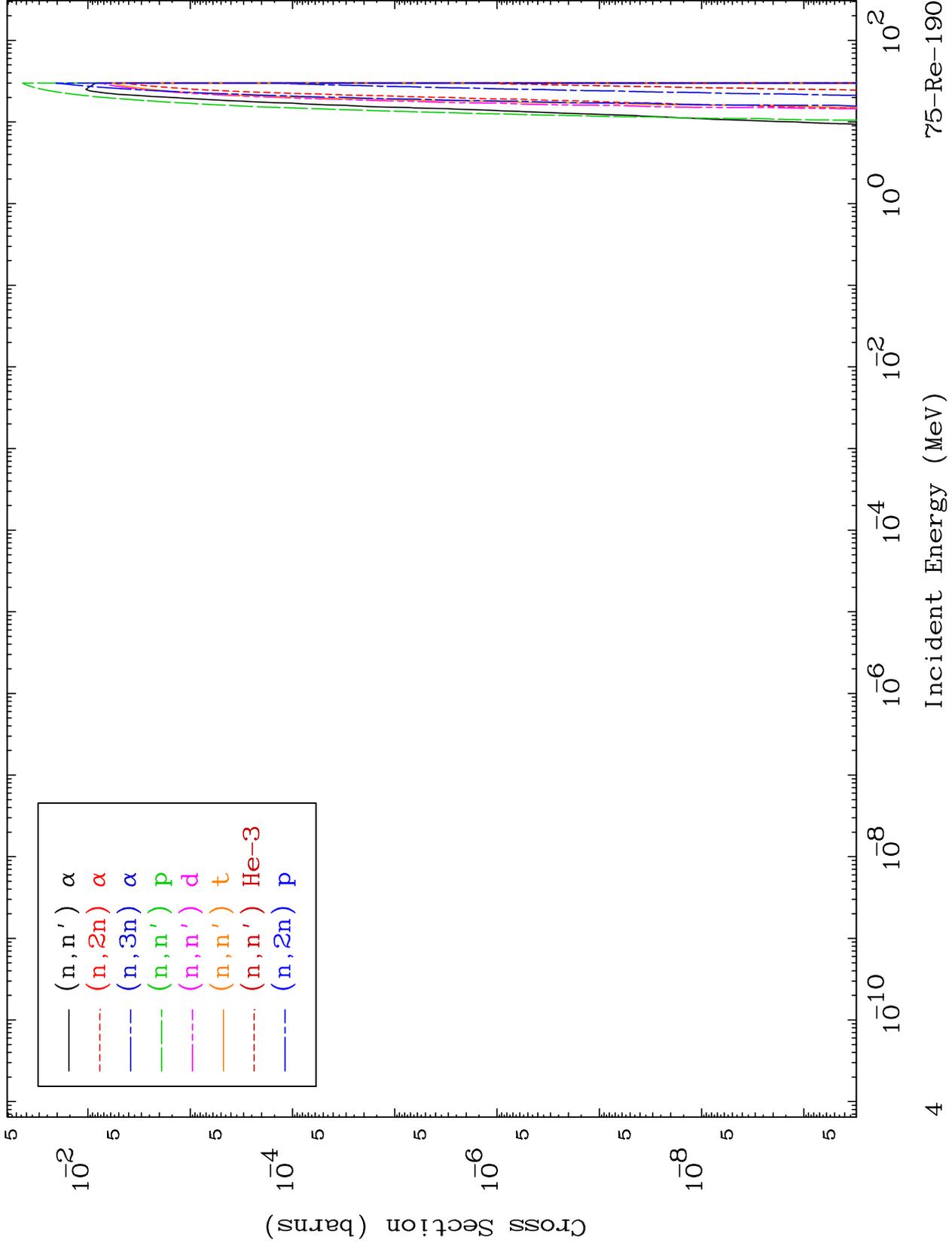
Incident Energy (MeV)

75-Re-190

MAT 7541

Charged Particle
293 Kelvin Cross Sections

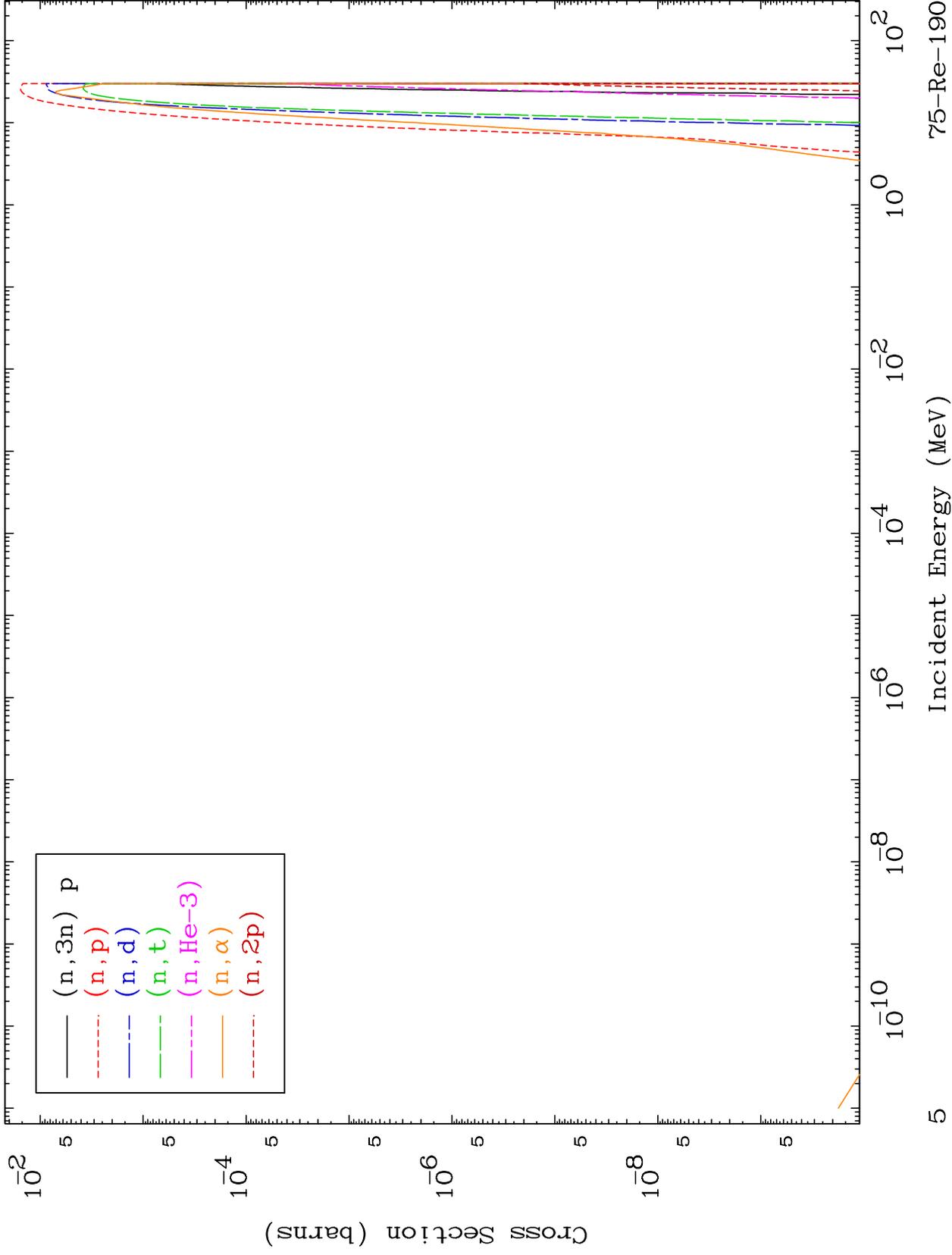
75-Re-190



MAT 7541

Charged Particle
293 Kelvin Cross Sections

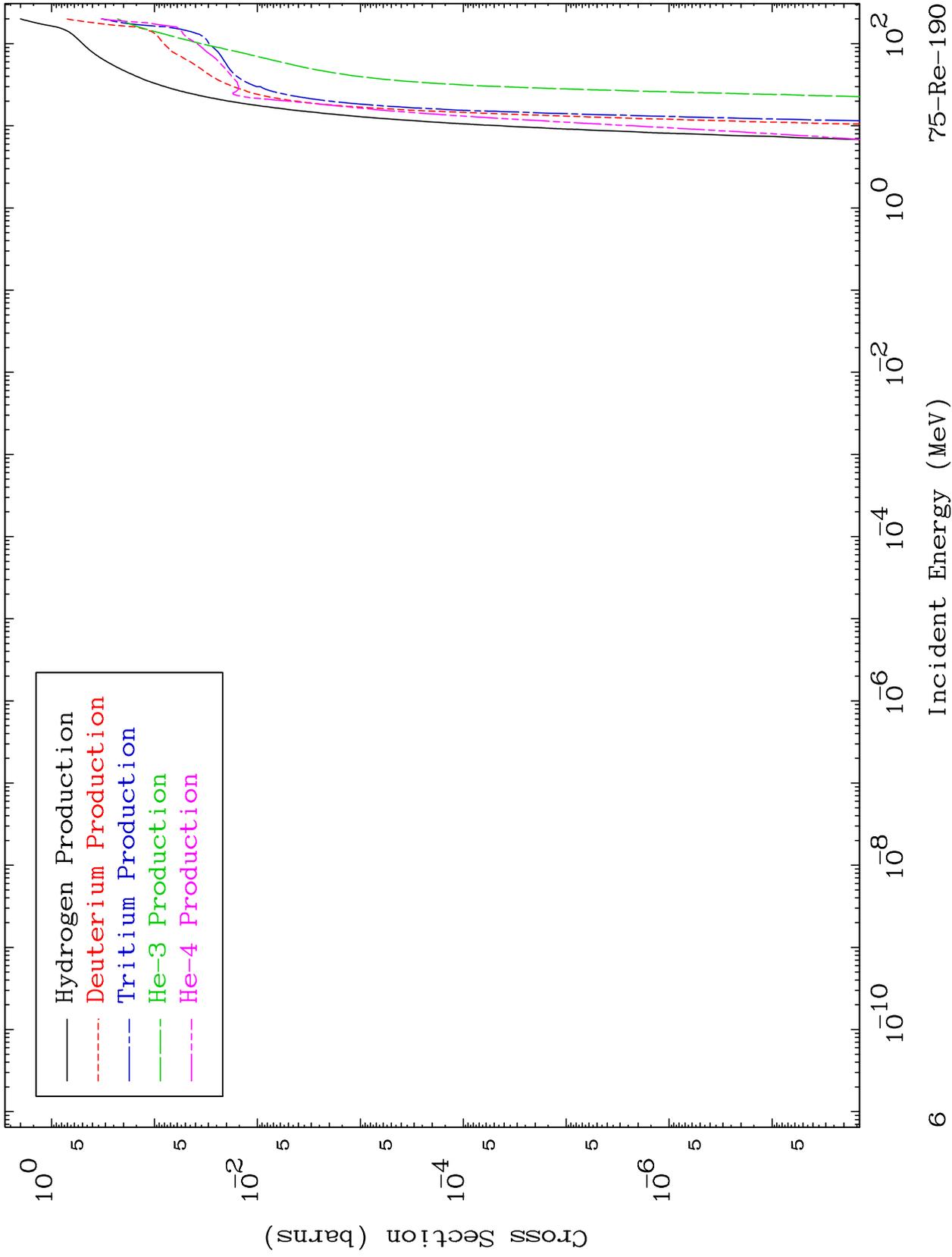
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MAT 7541

Particle Production
293 Kelvin Cross Sections

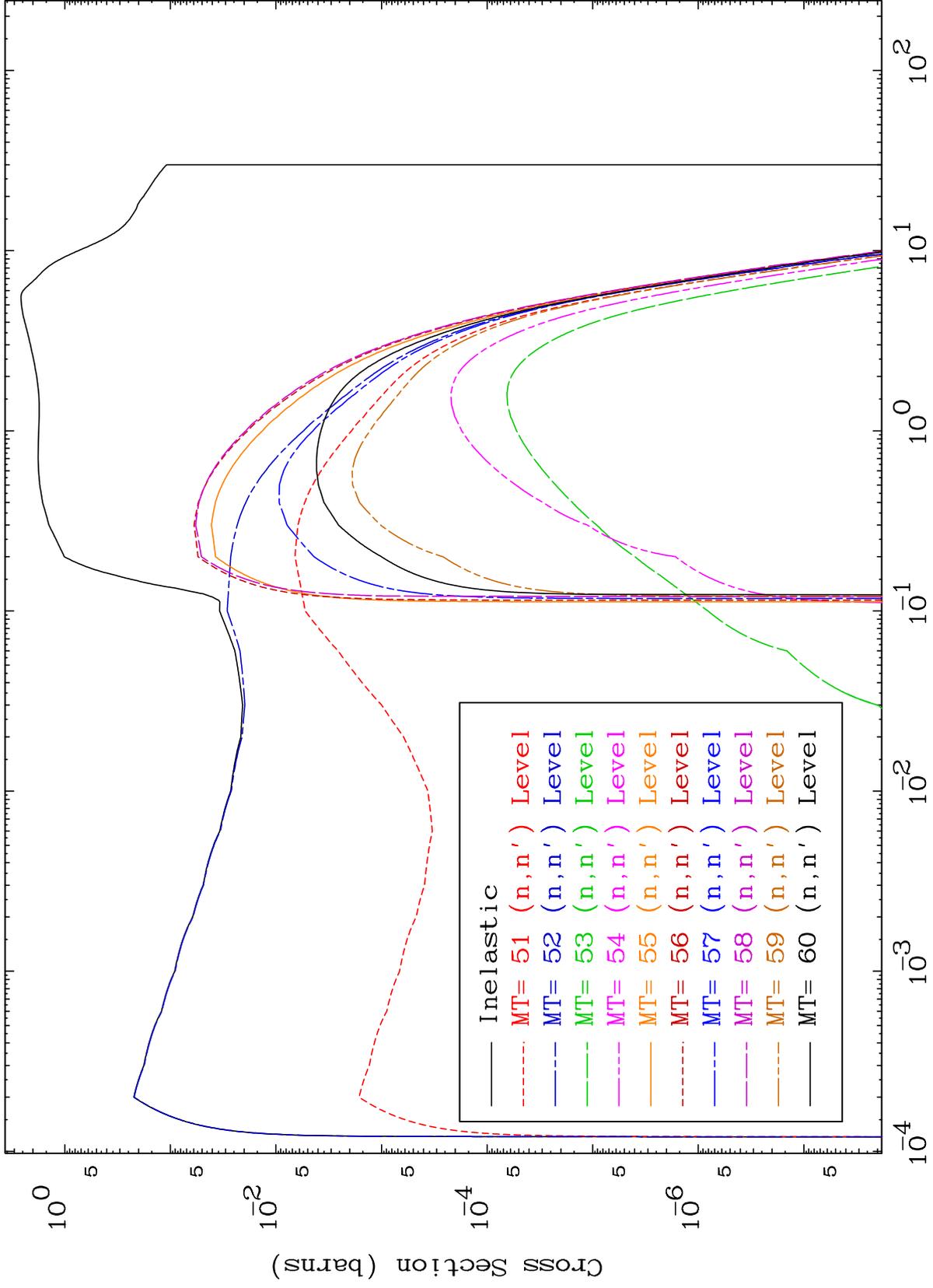
75-Re-190



MAT 7541

(n,n') Level
293 Kelvin Cross Sections

75-Re-190

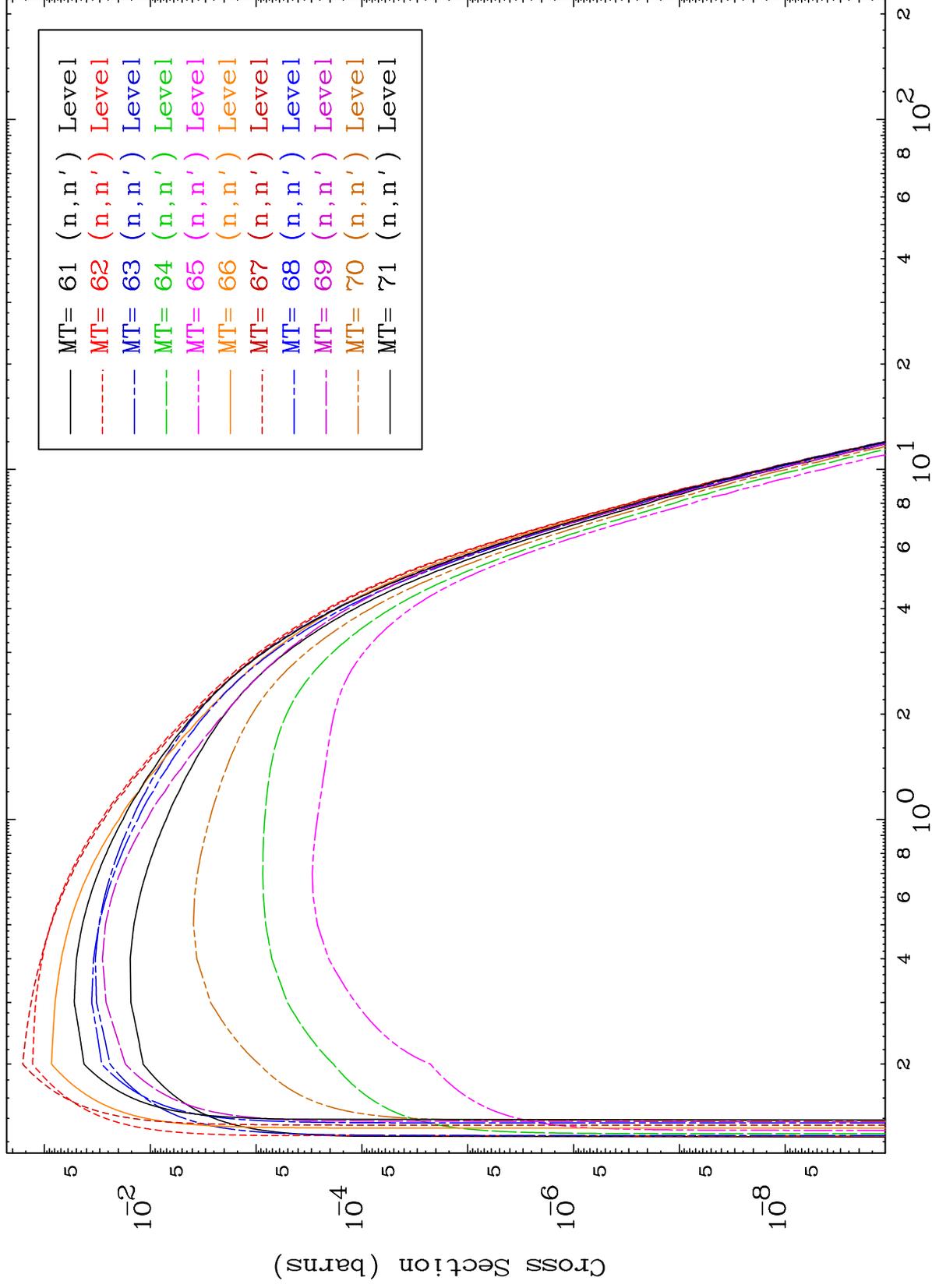


MAT 7541

(n,n') Level

75-Re-190

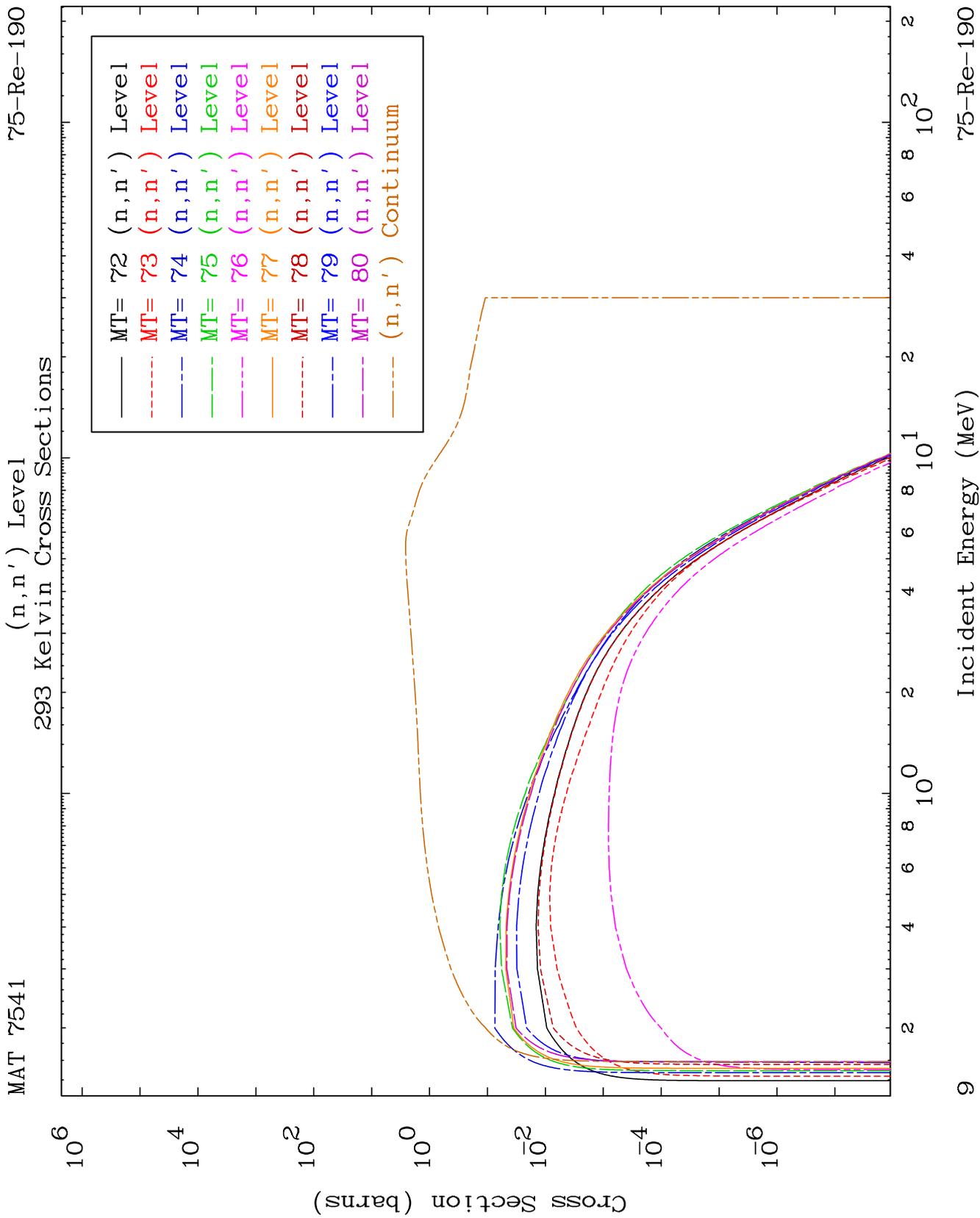
293 Kelvin Cross Sections



8

Incident Energy (MeV)

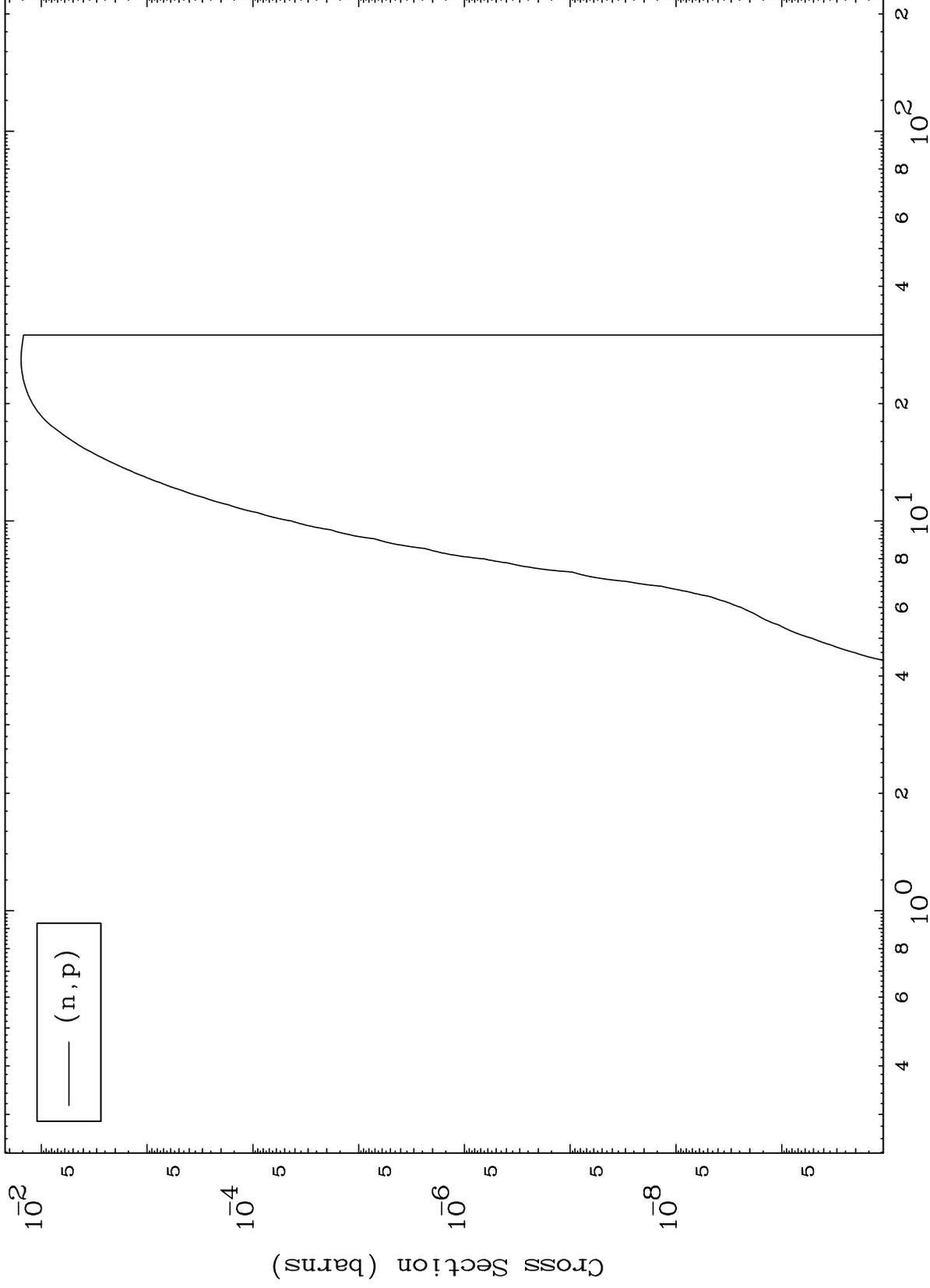
75-Re-190



MAT 7541

(n,p) Levels
293 Kelvin Cross Sections

75-Re-190



10

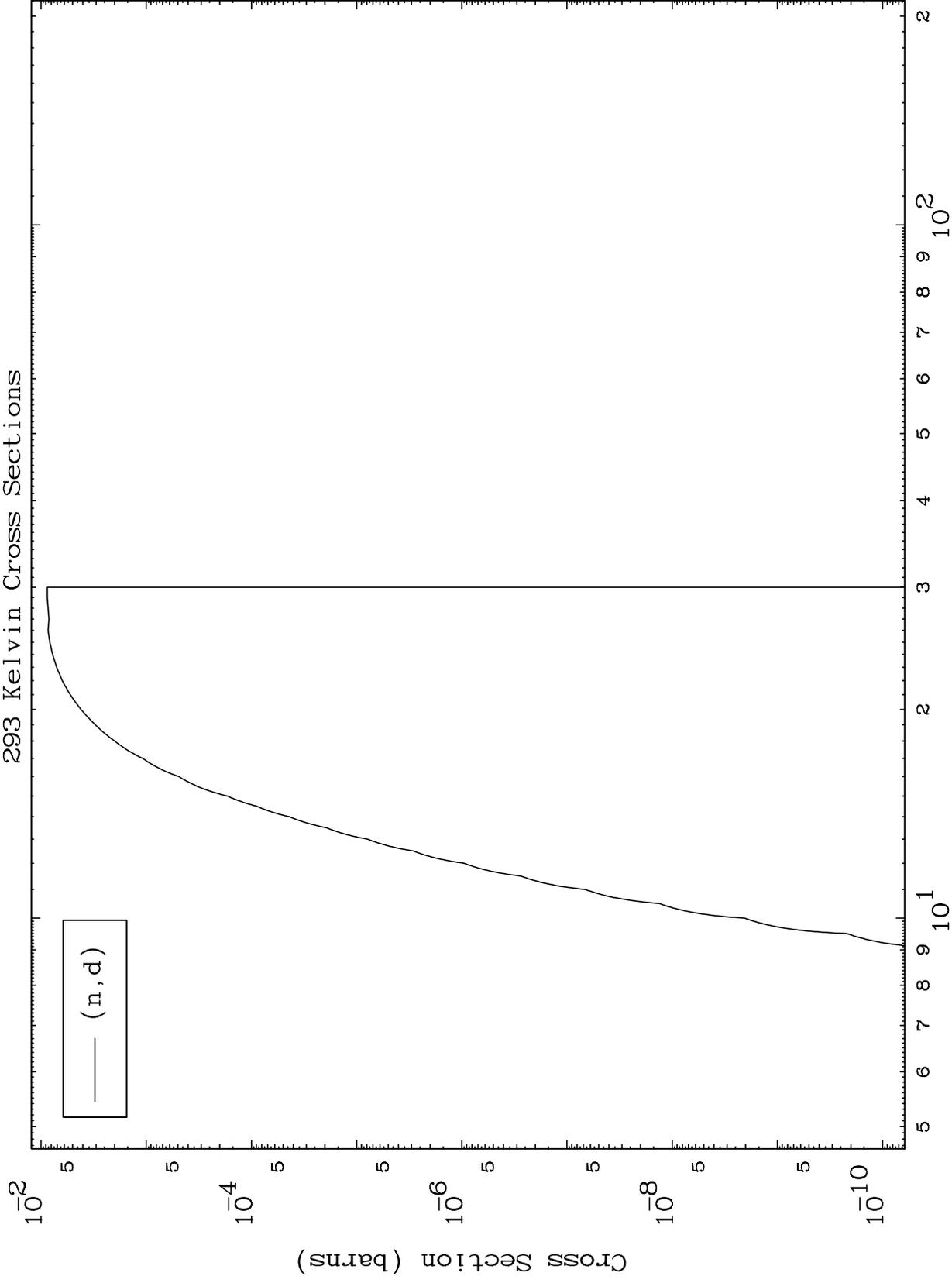
Incident Energy (MeV)

75-Re-190

MAT 7541

75-Re-190

(n,d) Levels
293 Kelvin Cross Sections



75-Re-190

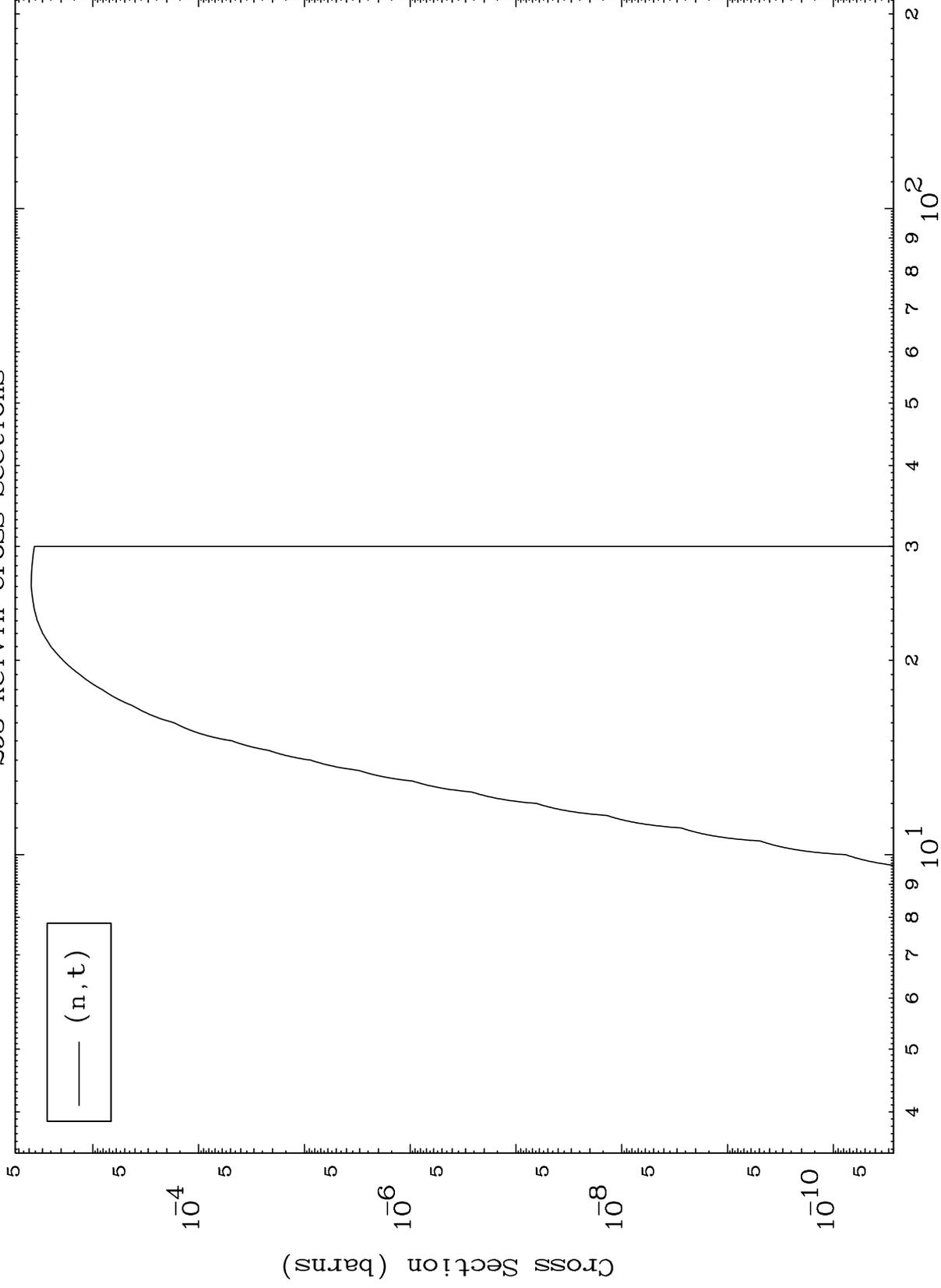
Incident Energy (MeV)

11

MAT 7541

(n,t) Levels
293 Kelvin Cross Sections

75-Re-190



12

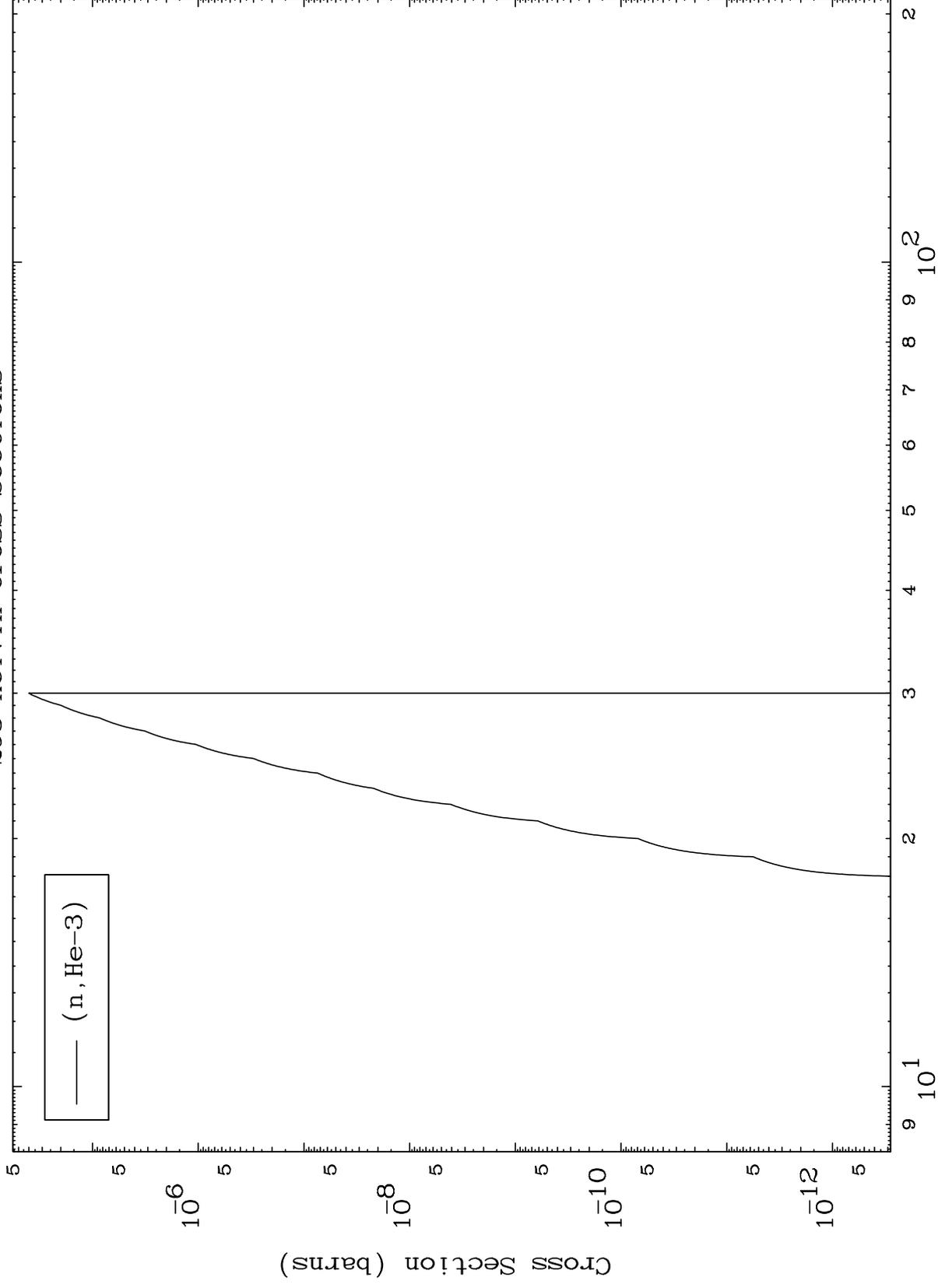
Incident Energy (MeV)

75-Re-190

MAT 7541

(n,He3) Levels
293 Kelvin Cross Sections

75-Re-190



13

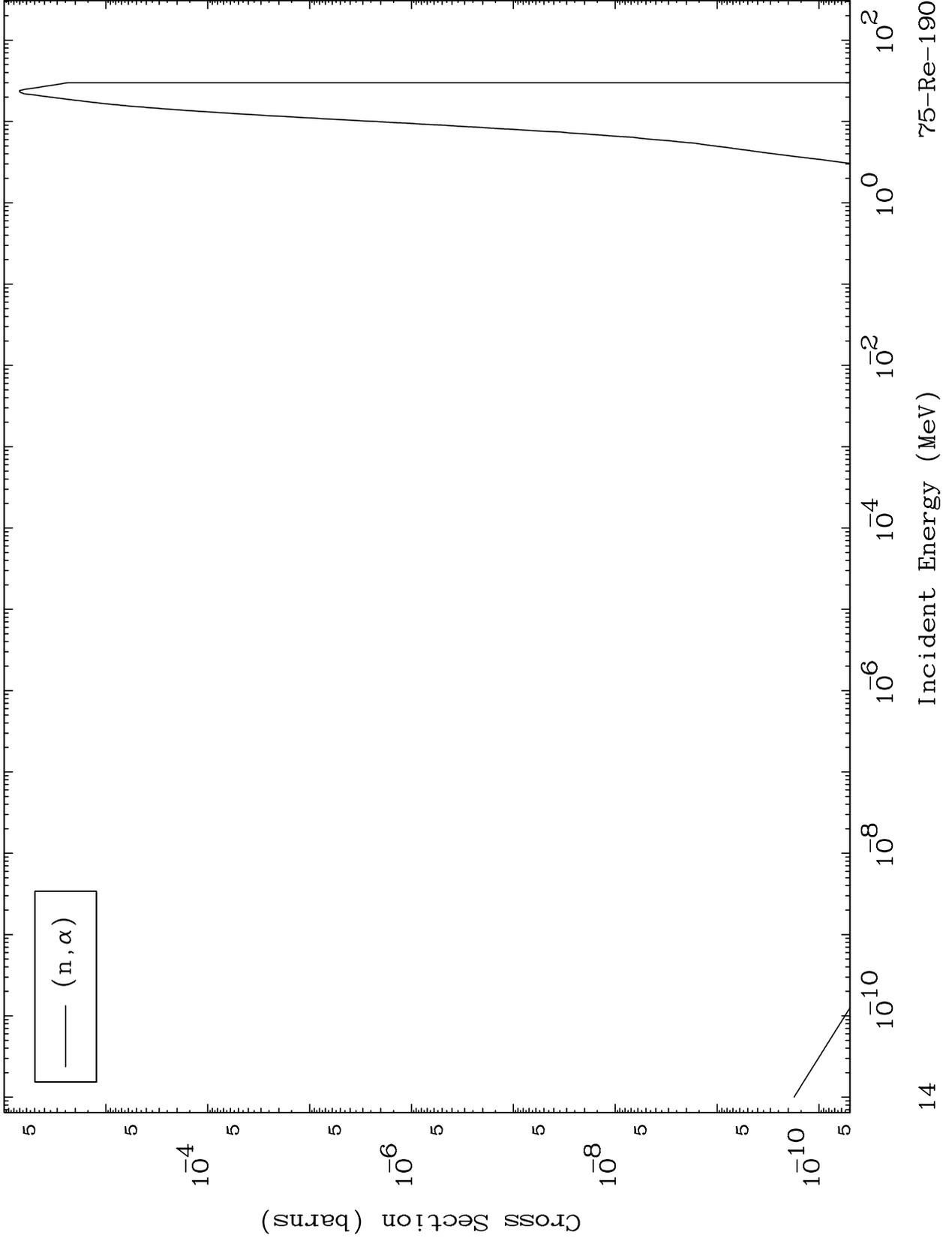
Incident Energy (MeV)

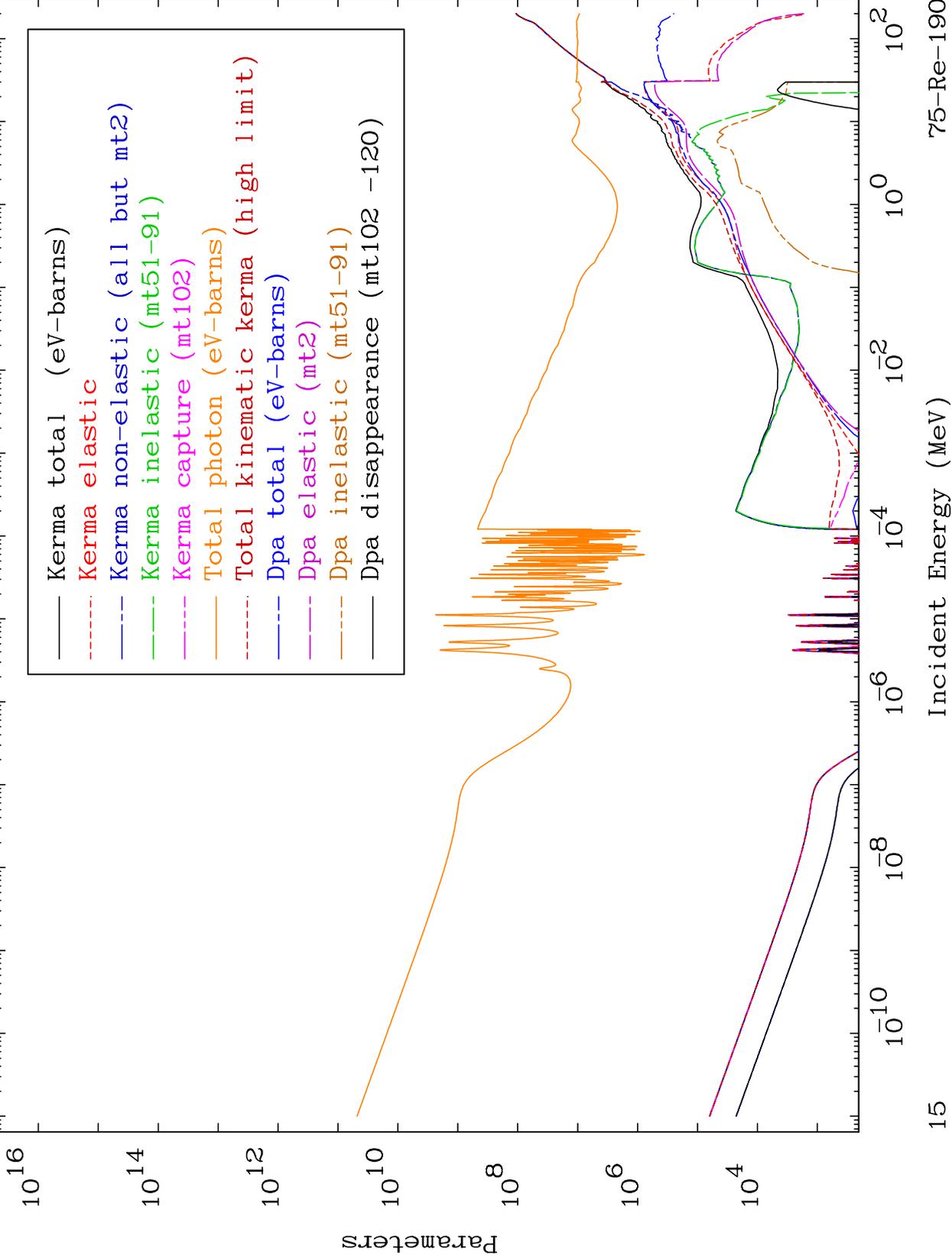
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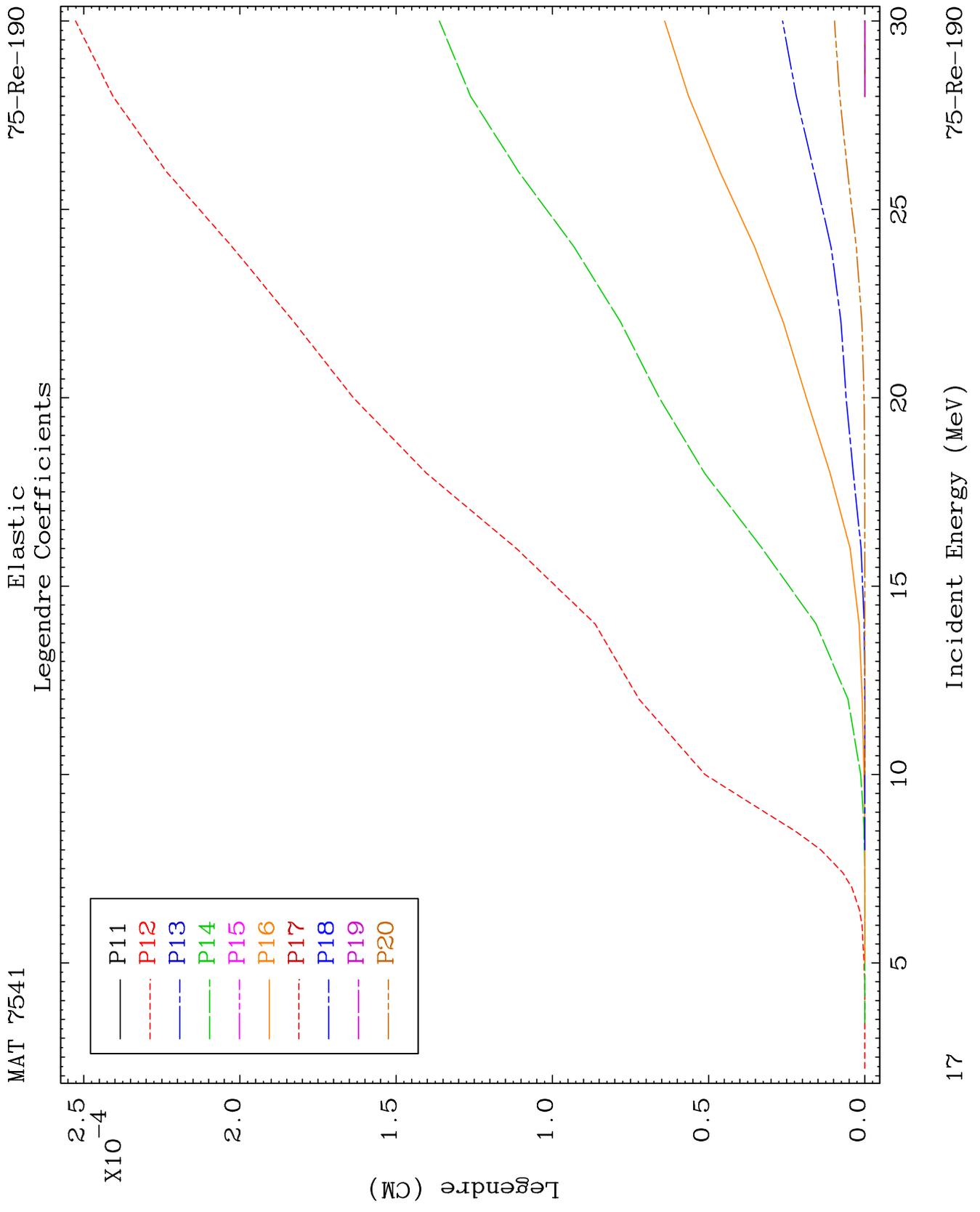
MAT 7541

(n, α) Levels
293 Kelvin Cross Sections

75-Re-190



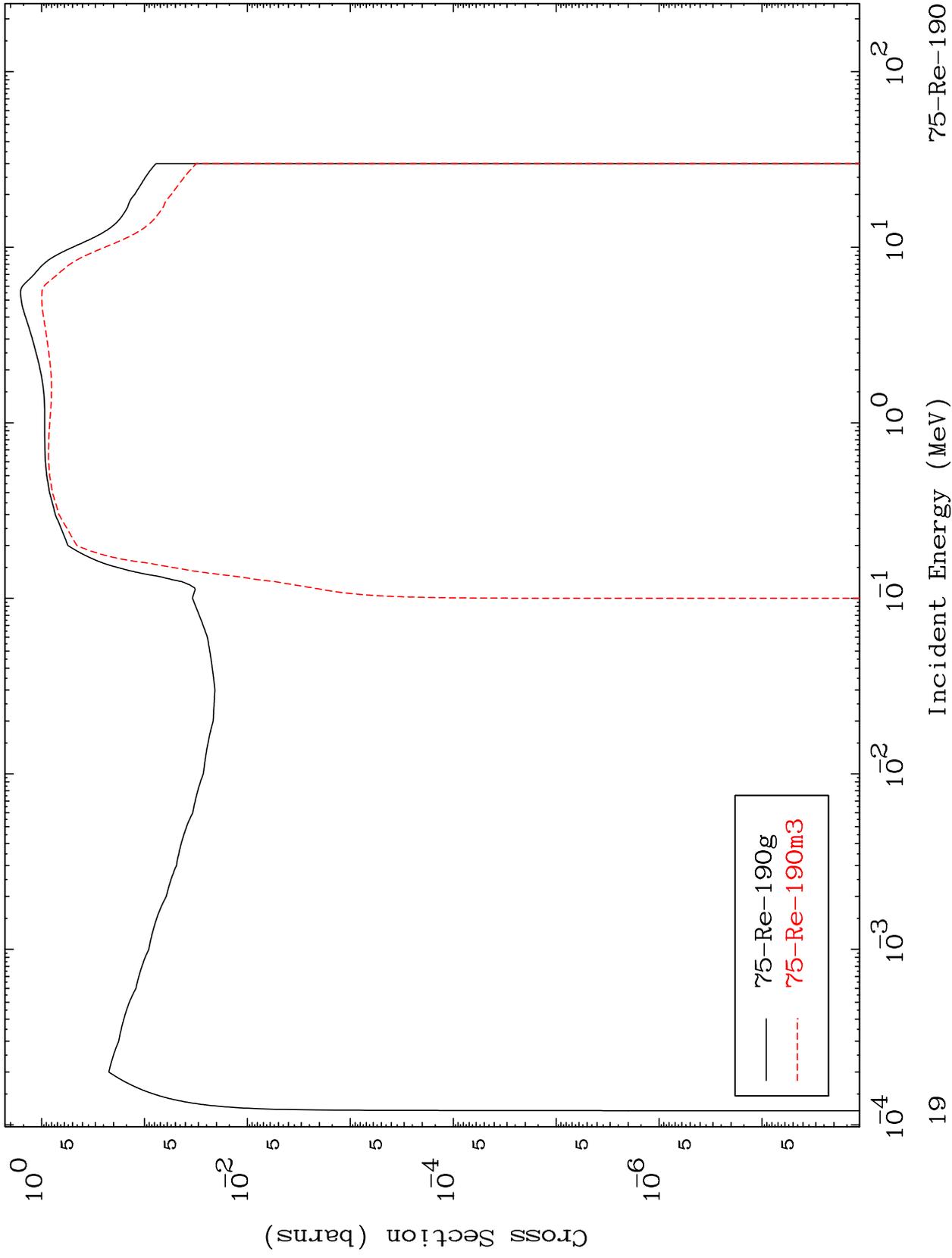




MAT 7541

75-Re-190

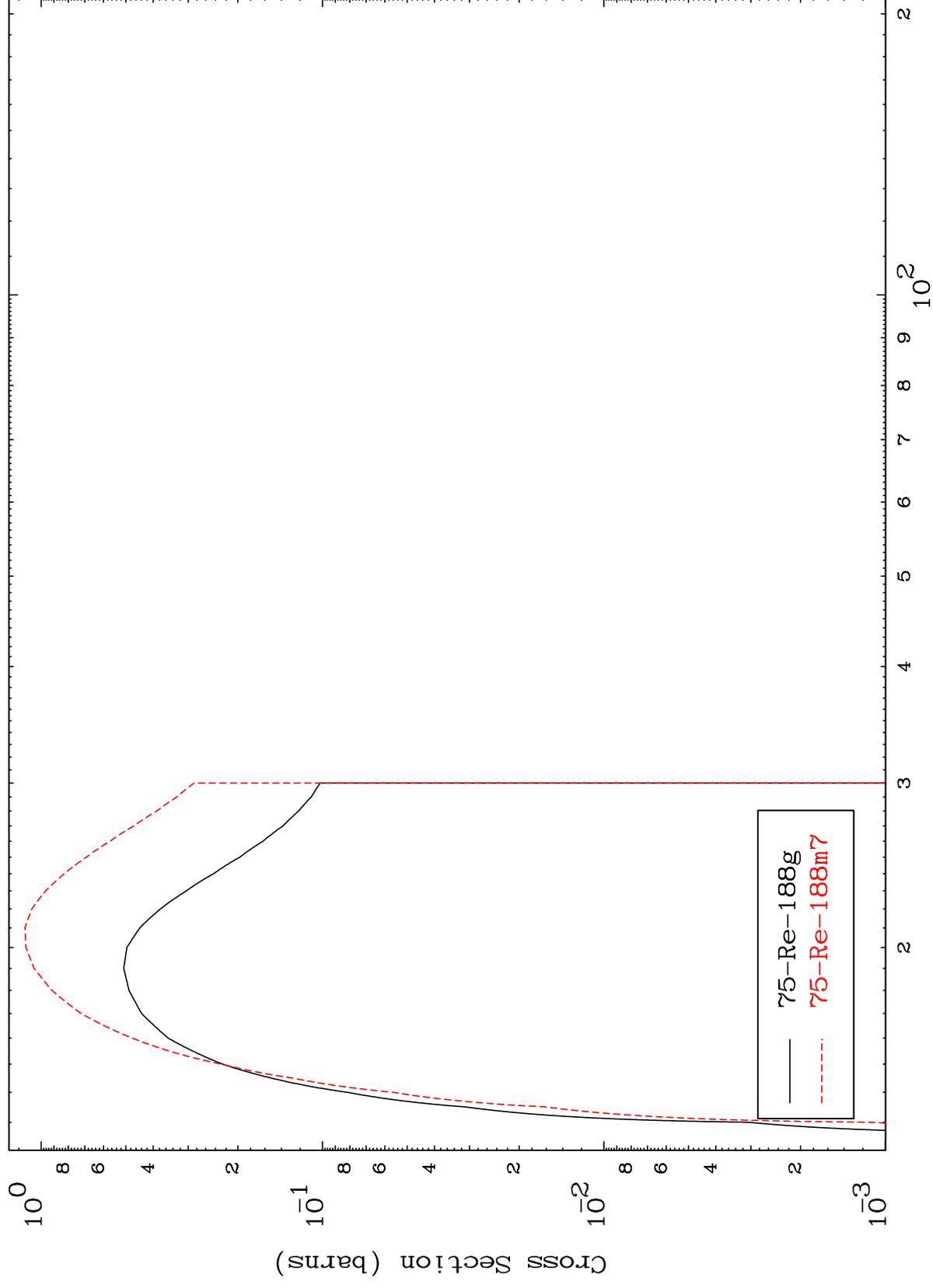
Inelastic
Radionuclide Production Cross Section



MAT 7541

75-Re-190

(n,3n)
Radionuclide Production Cross Section



20

Incident Energy (MeV)

75-Re-190

MAT 7541

Fission

75-Re-190

Radionuclide Production Cross Section

