

Program EVALPLOT
(Version 2017-1)

by

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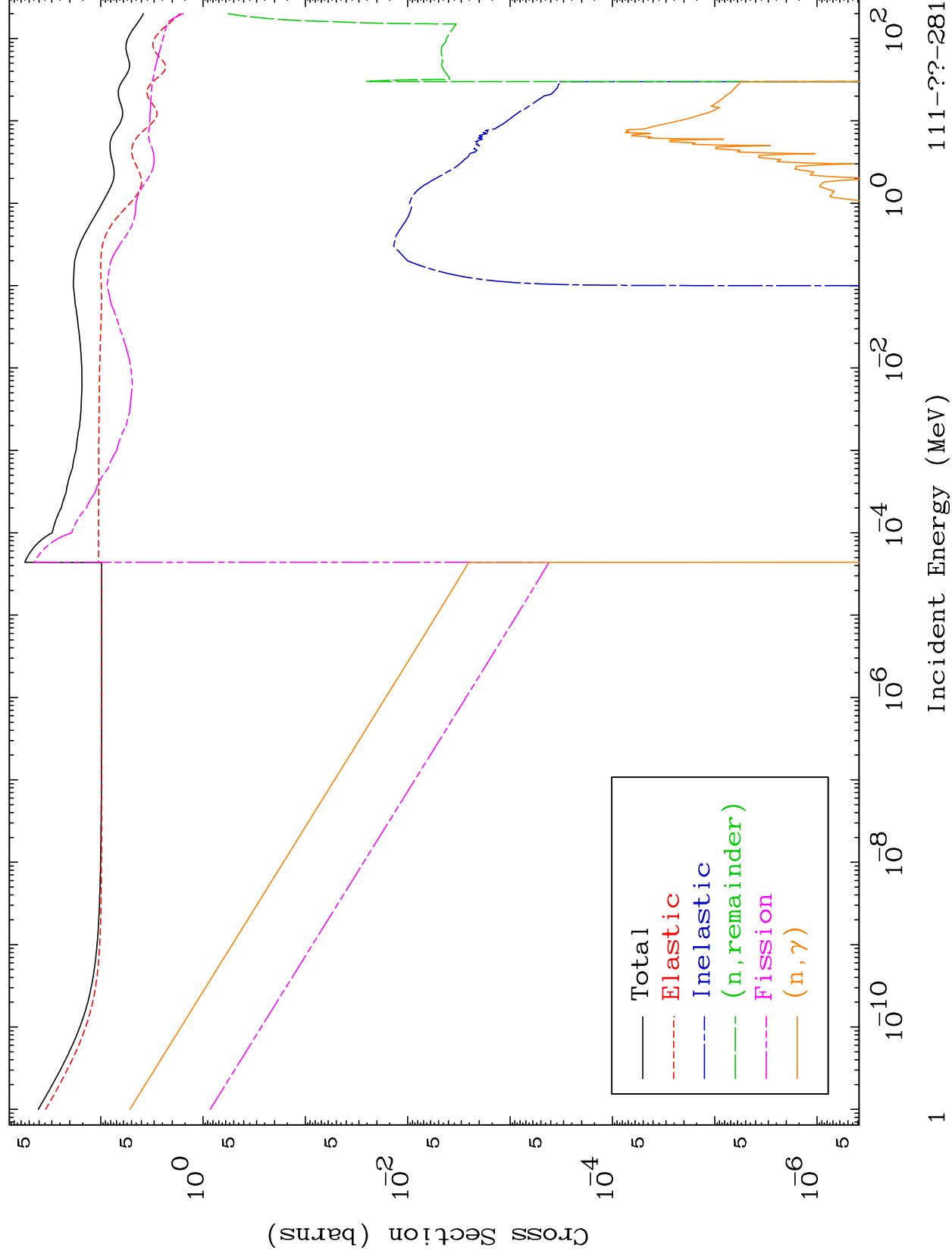
Press Mouse Button to Start

MAT 1181

Major

293 Kelvin Cross Sections

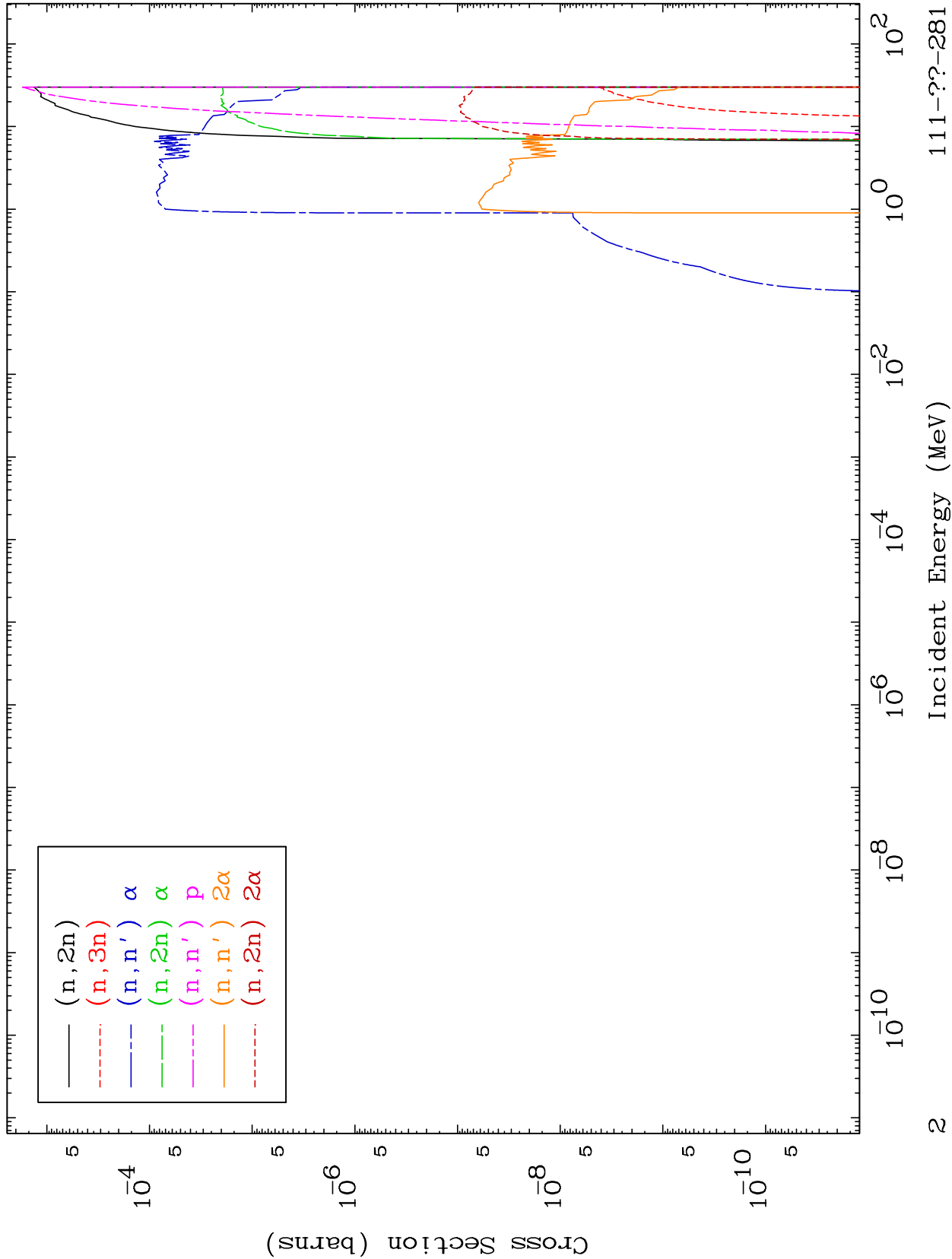
111-??-281

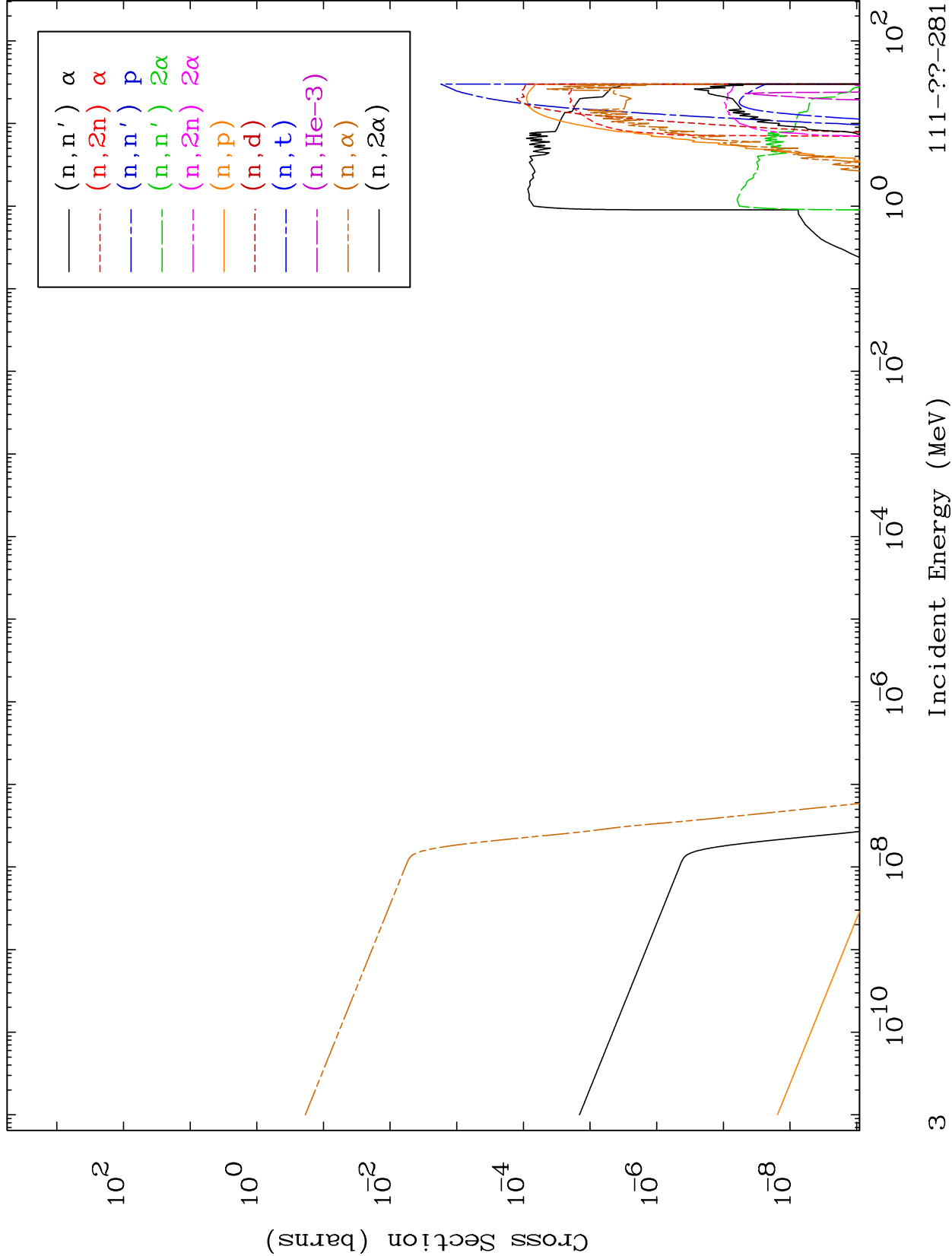


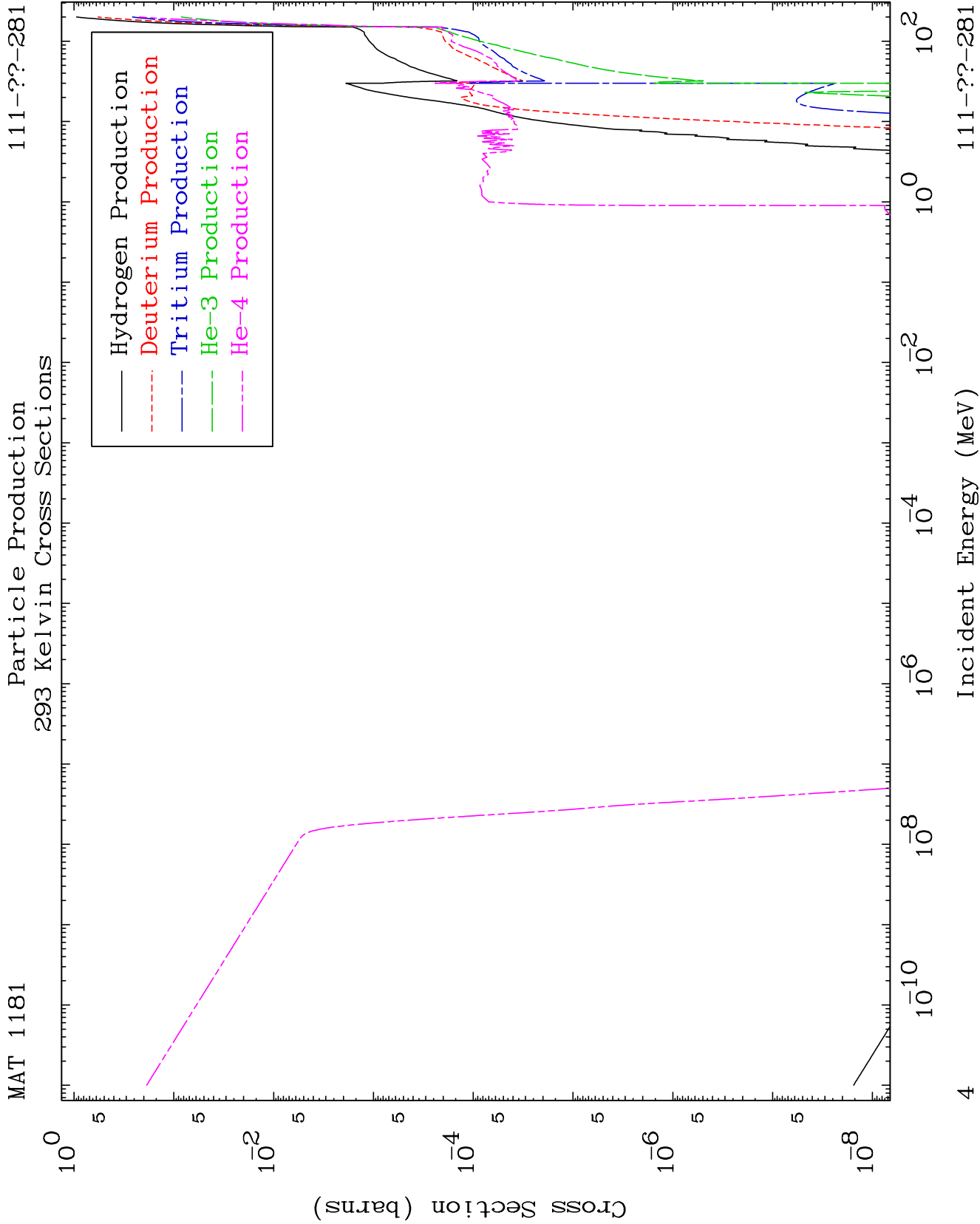
MAT 1181

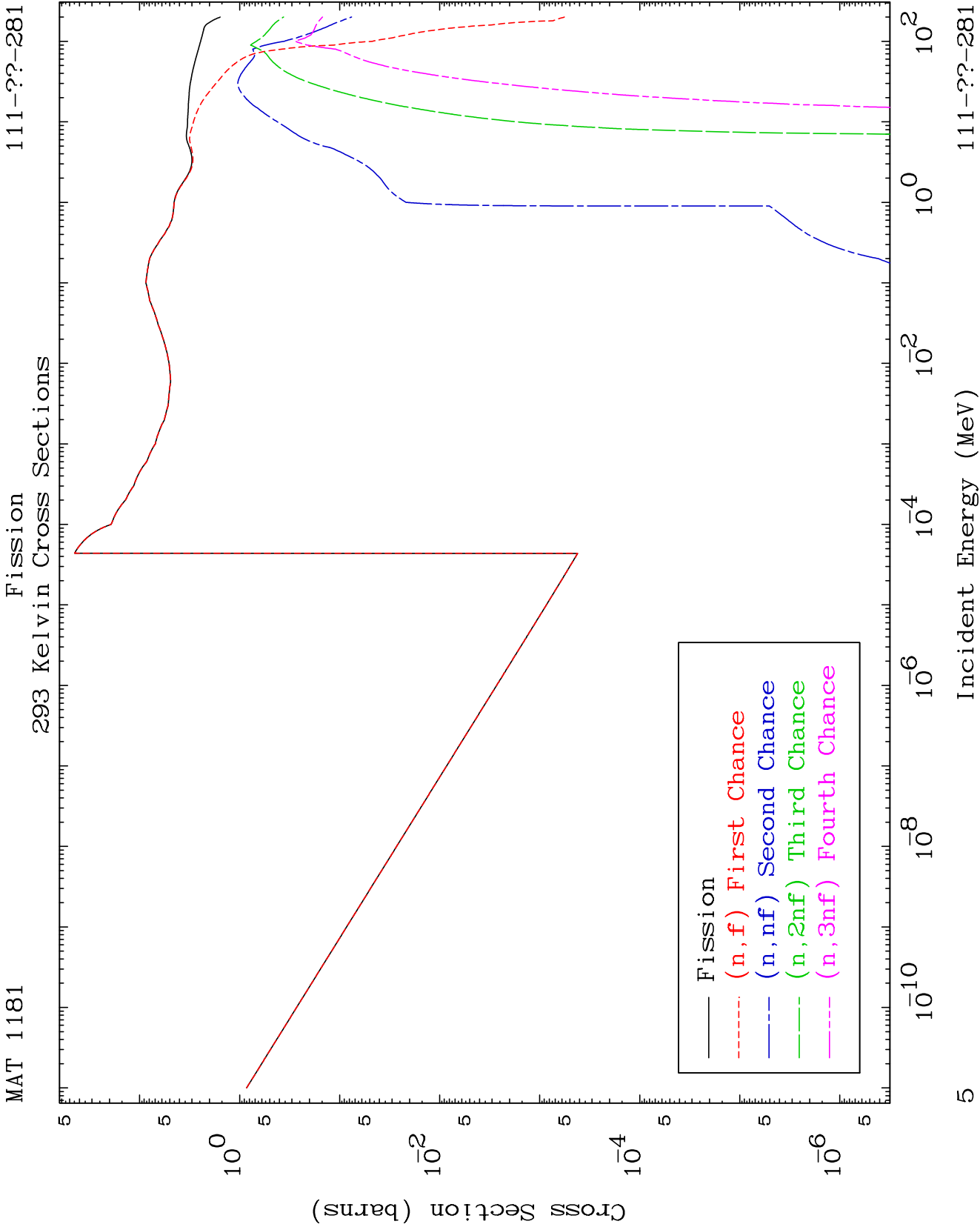
Neutron Production
293 Kelvin Cross Sections

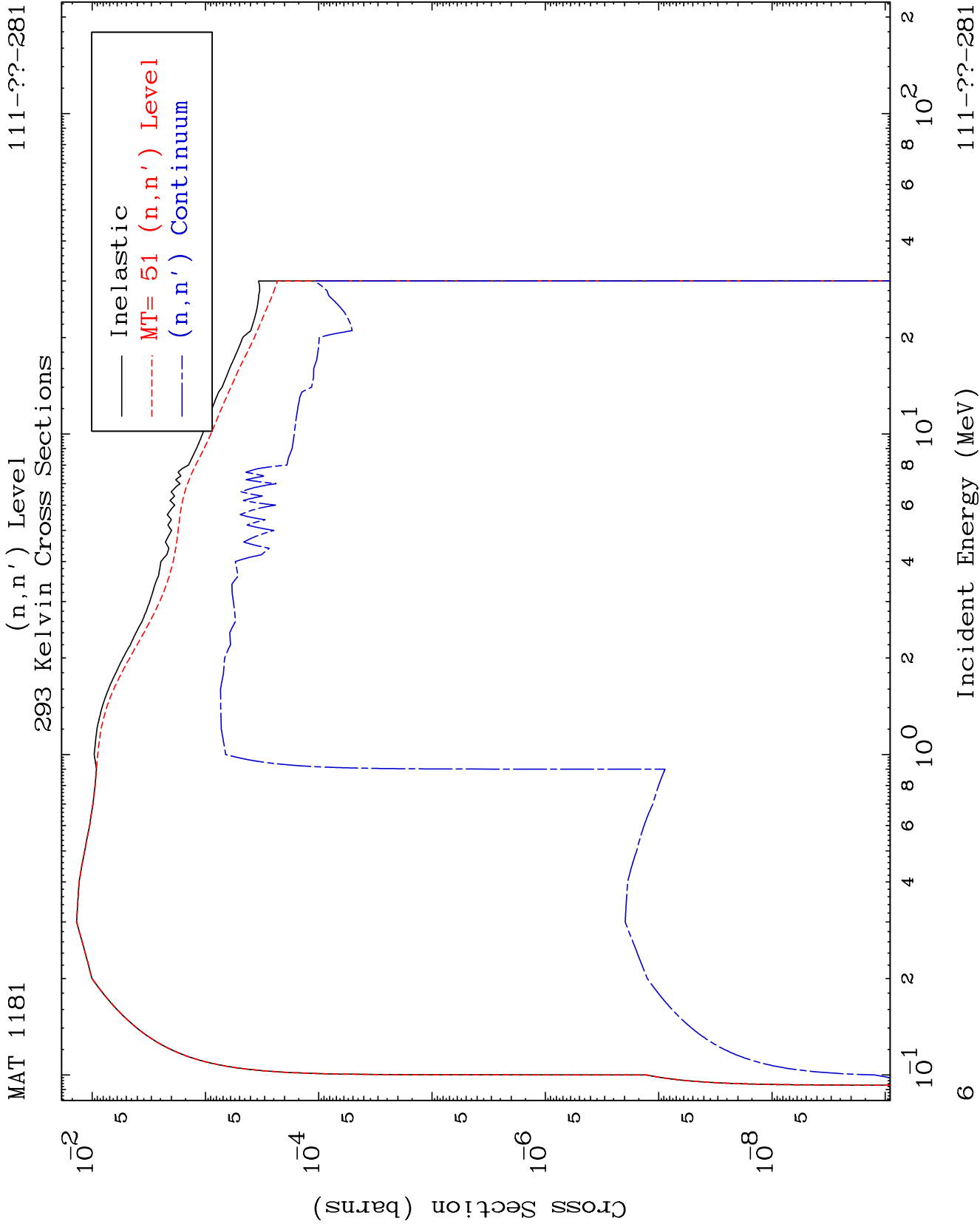
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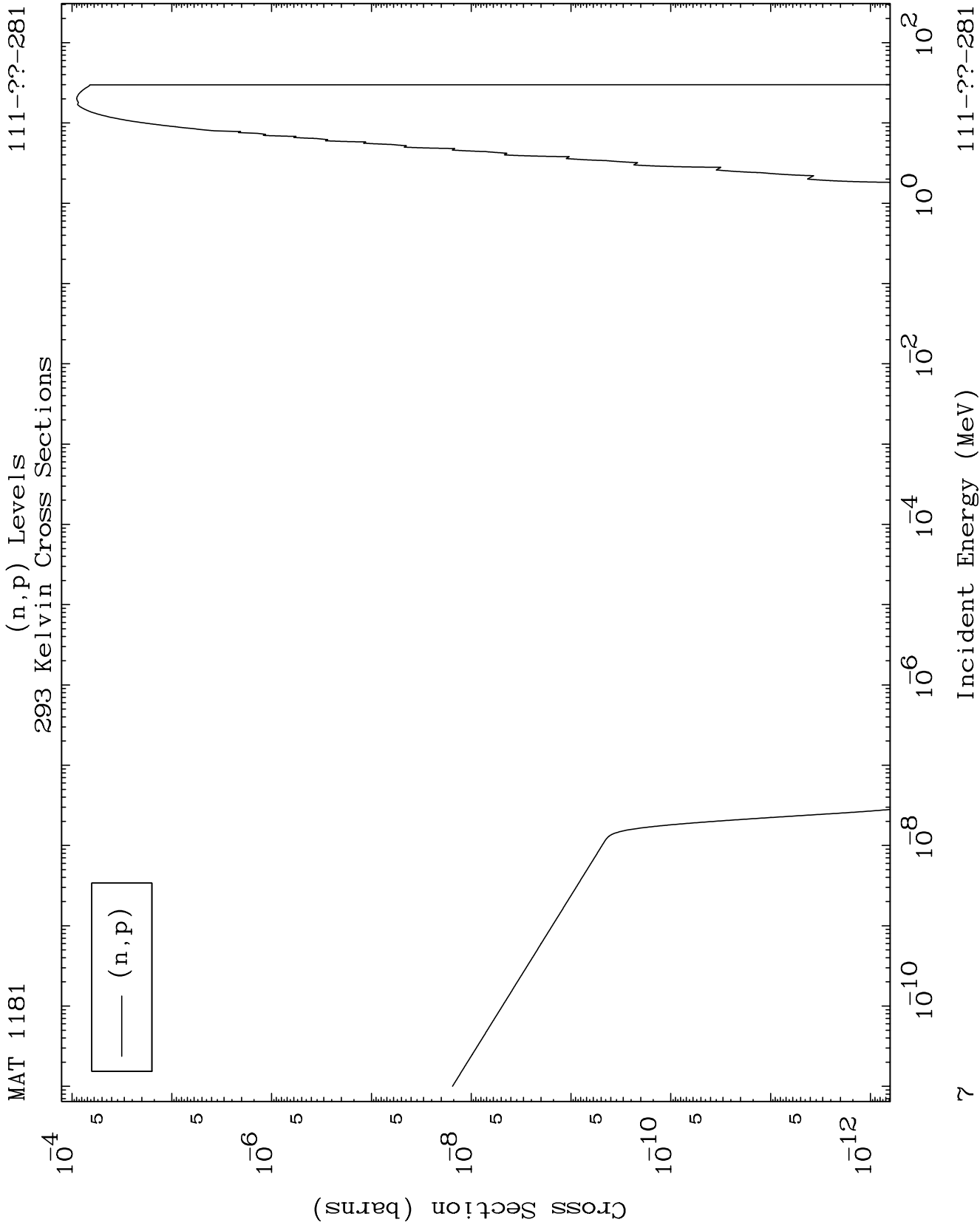








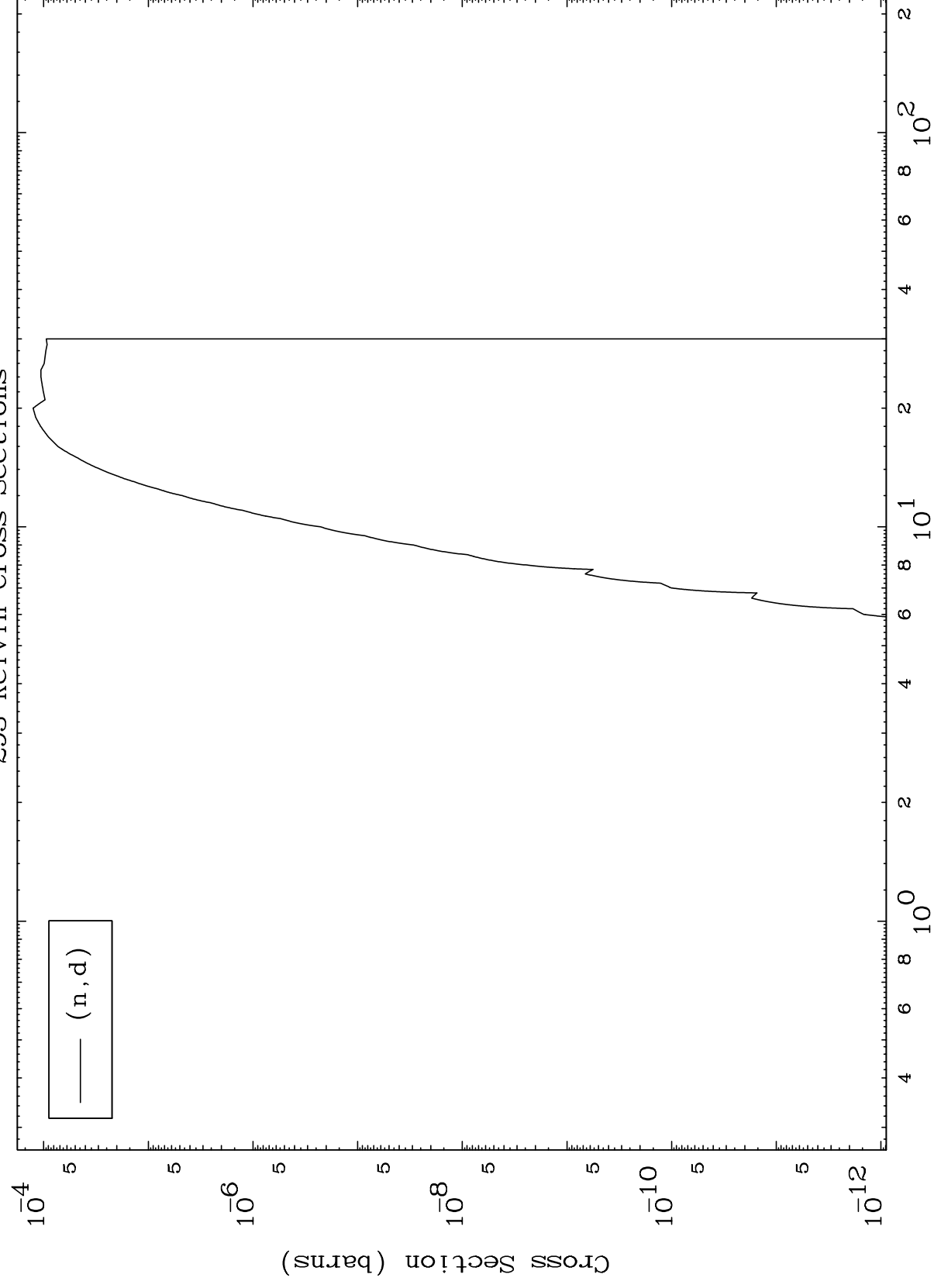




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(n,d) Levels
293 Kelvin Cross Sections

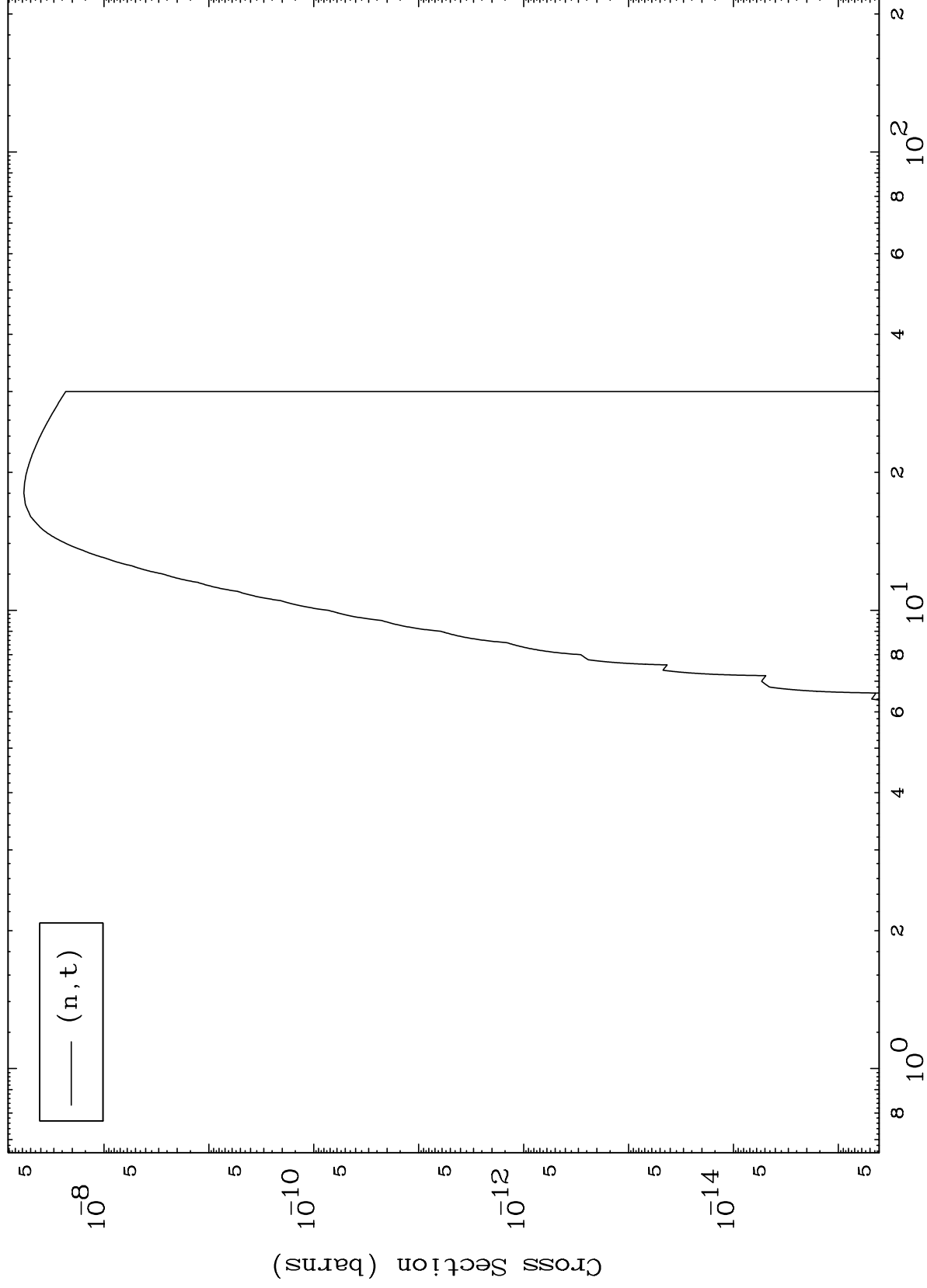


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(n,t) Levels
293 Kelvin Cross Sections



9

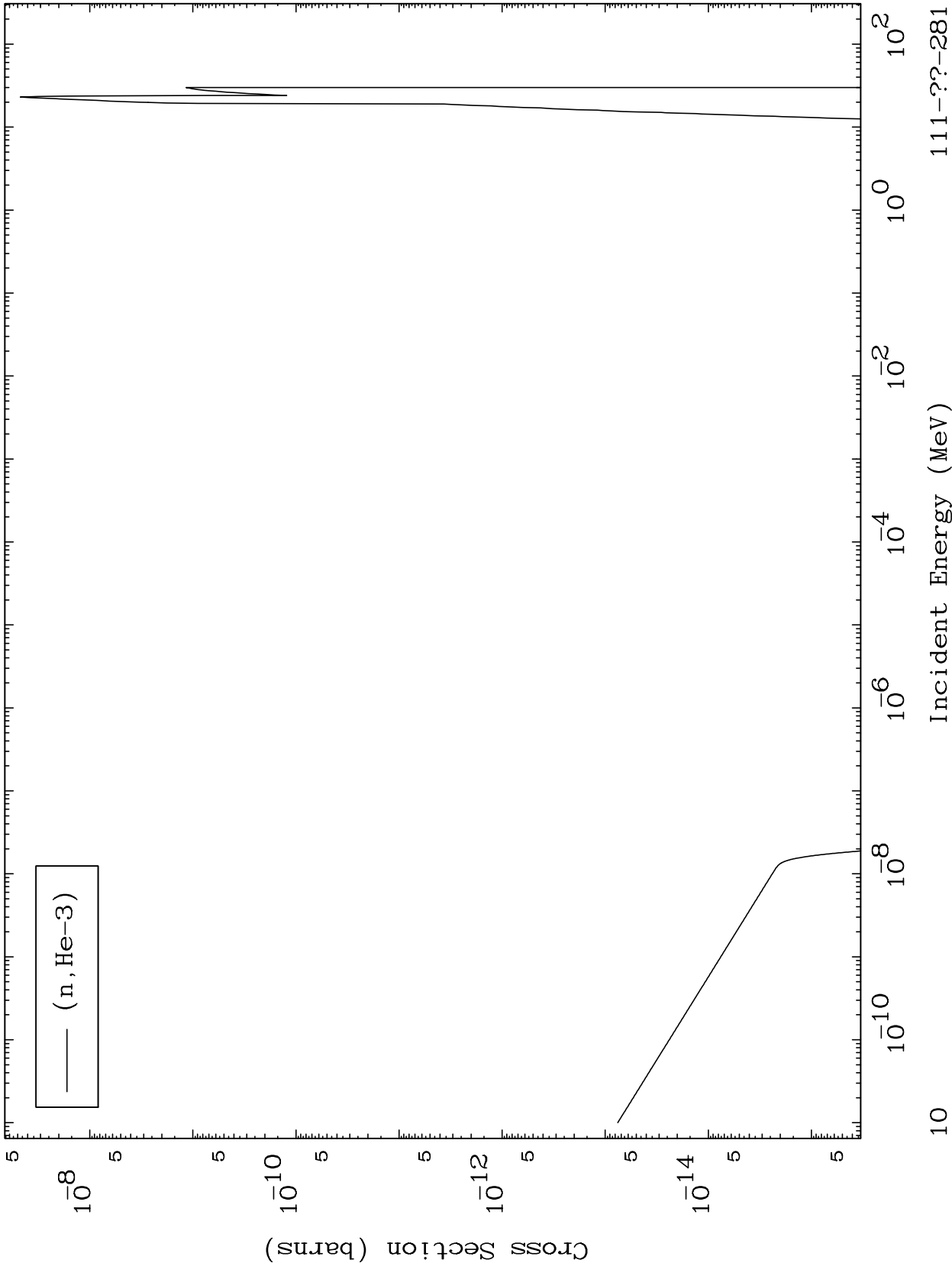
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Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

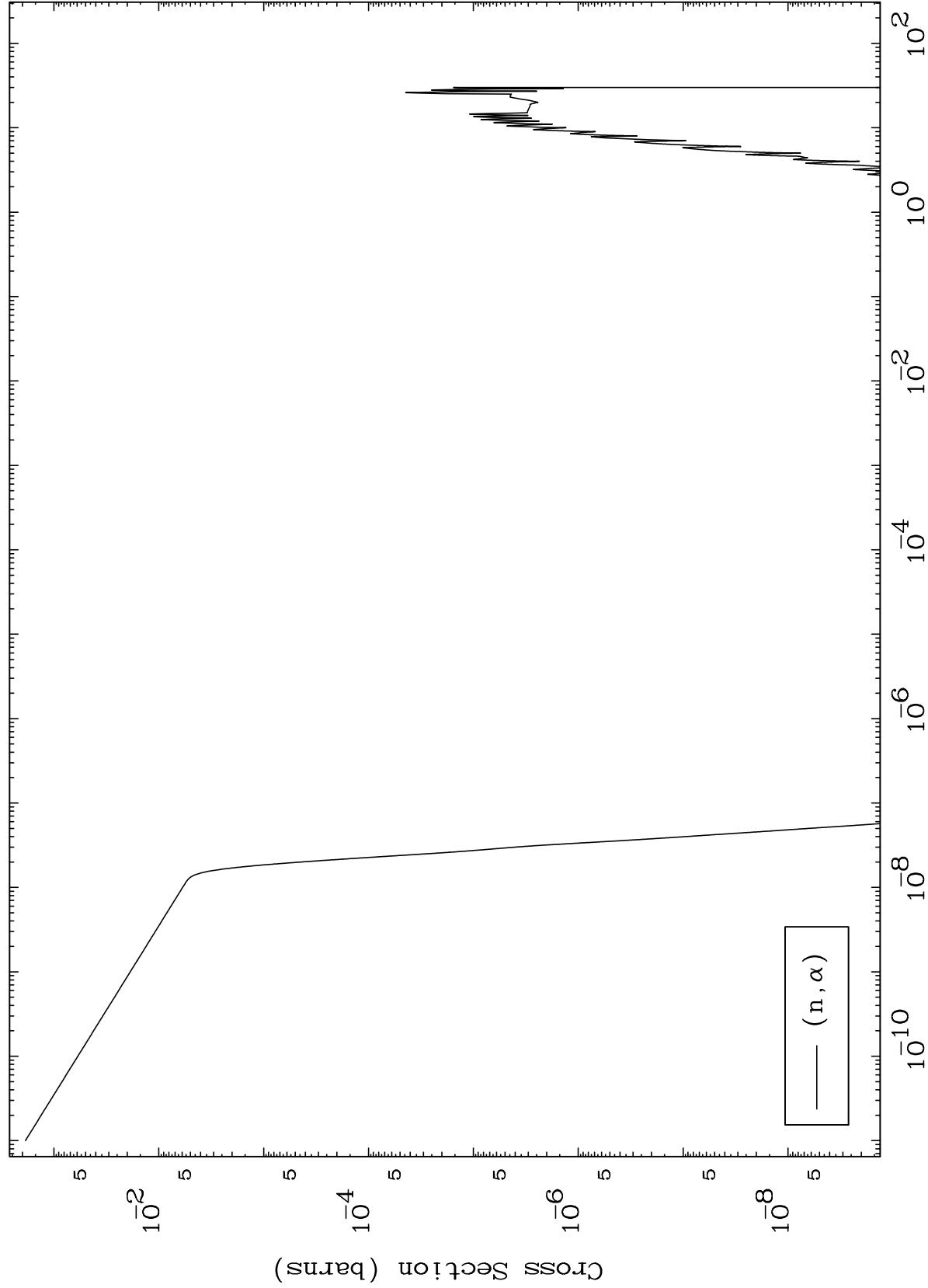
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(n, α) Levels
293 Kelvin Cross Sections



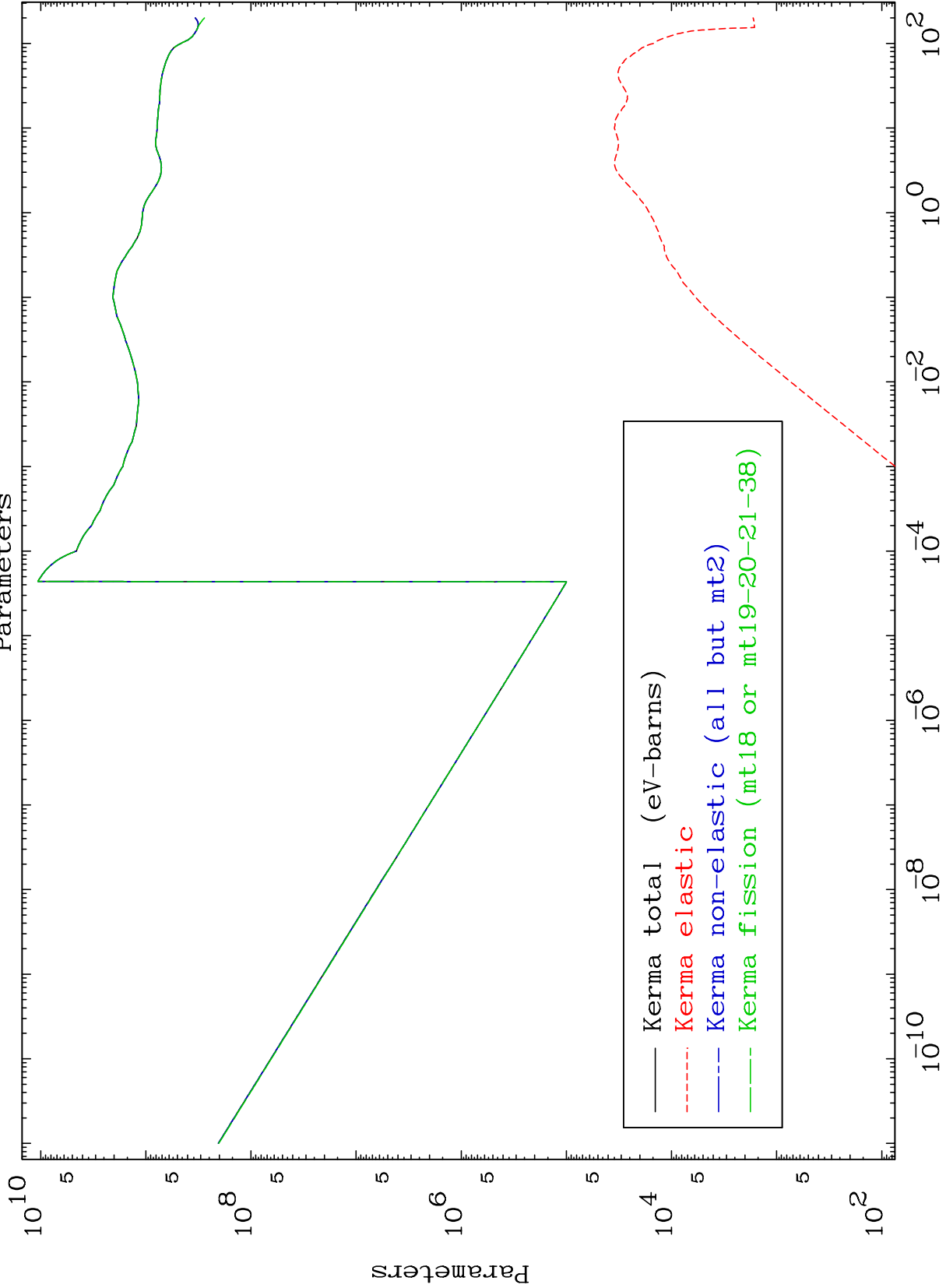
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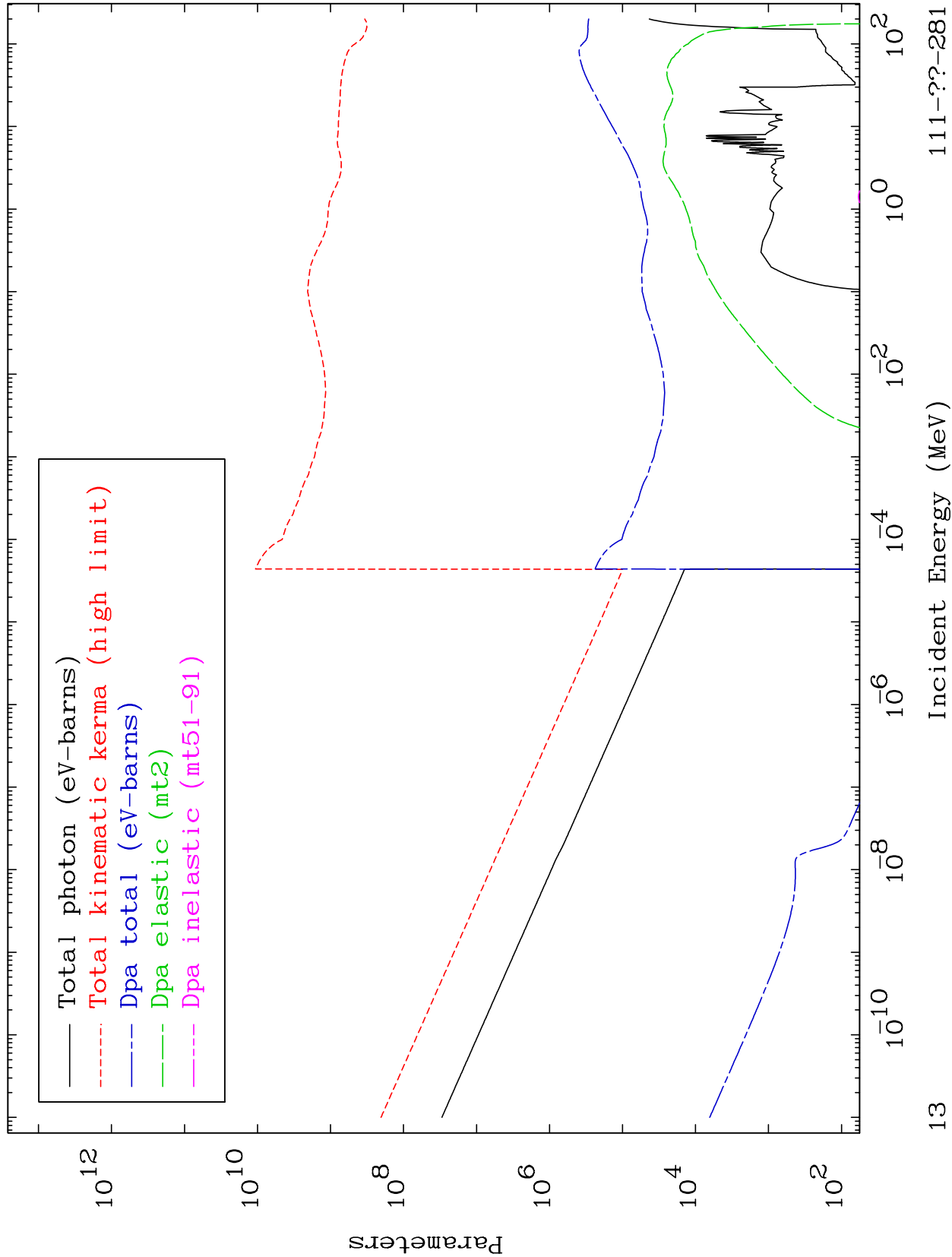
MAT 1181

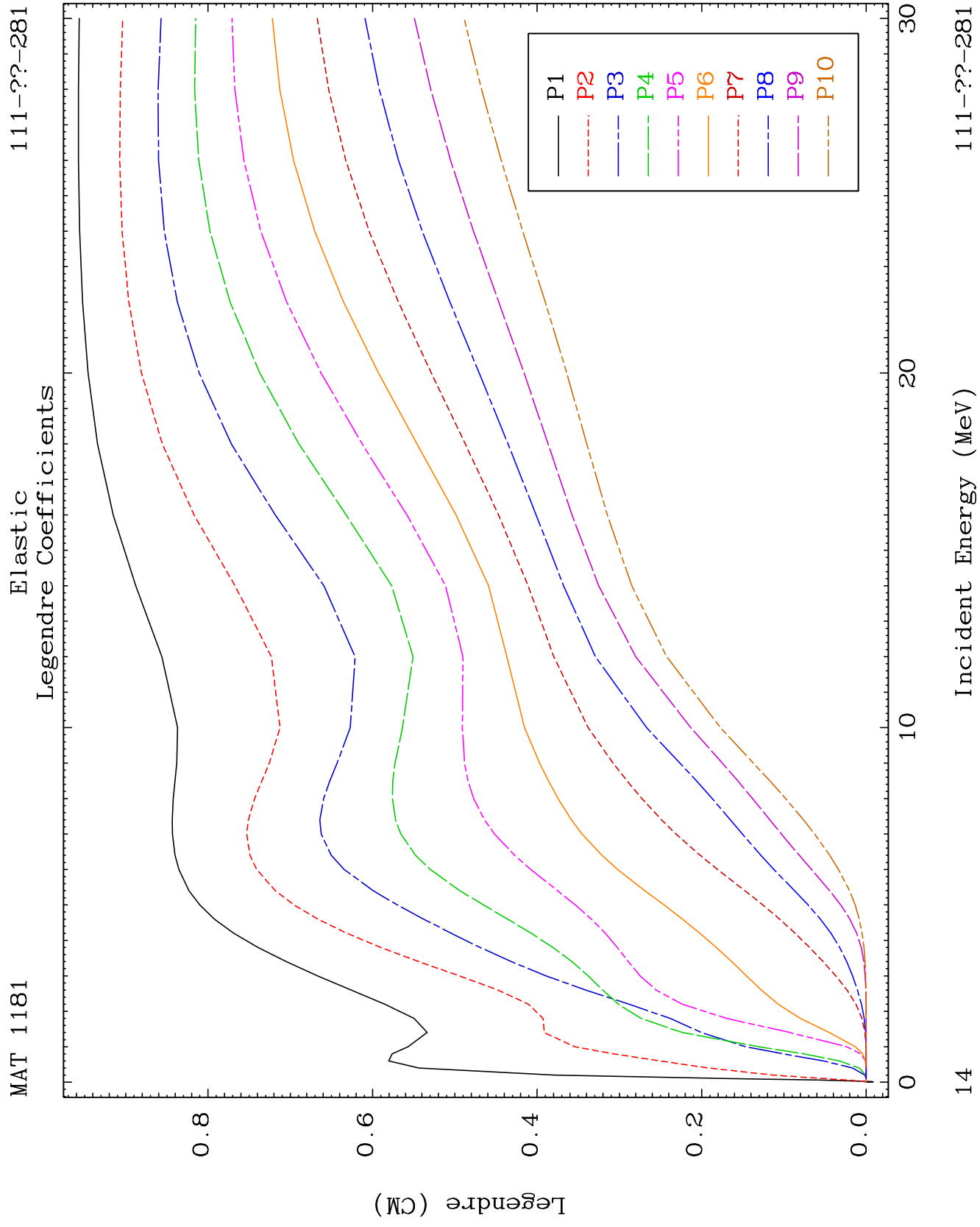
Energy Release
Parameters

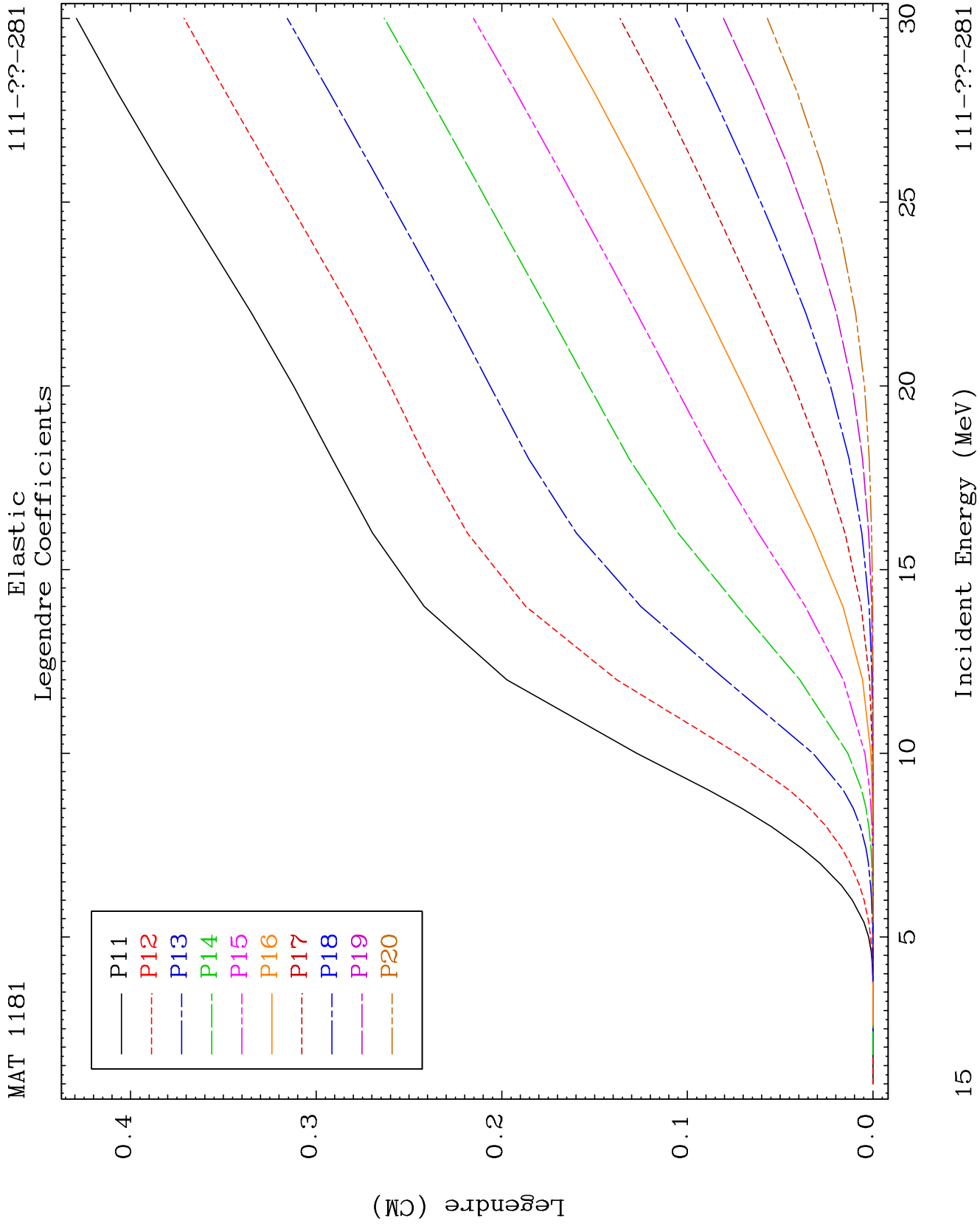
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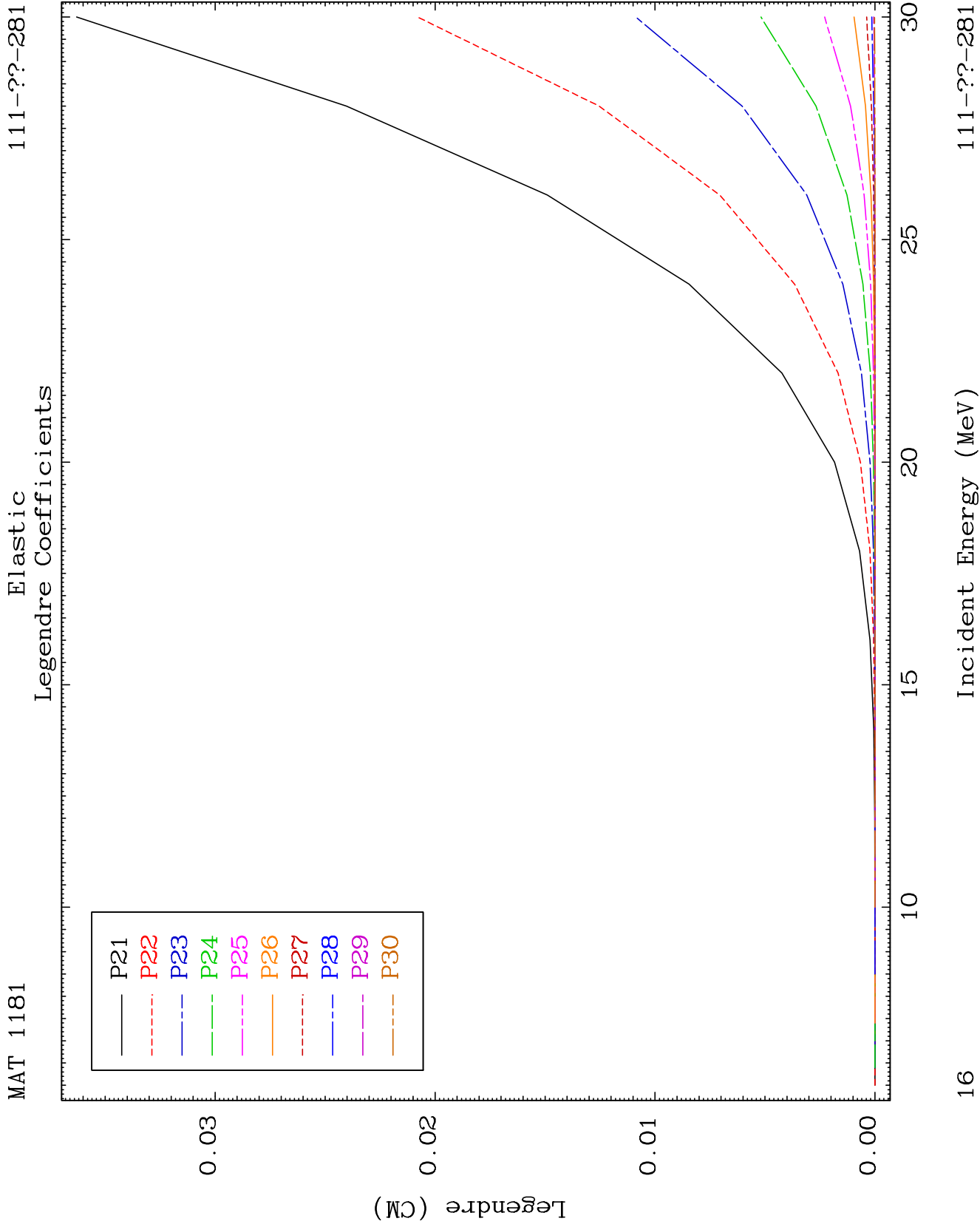


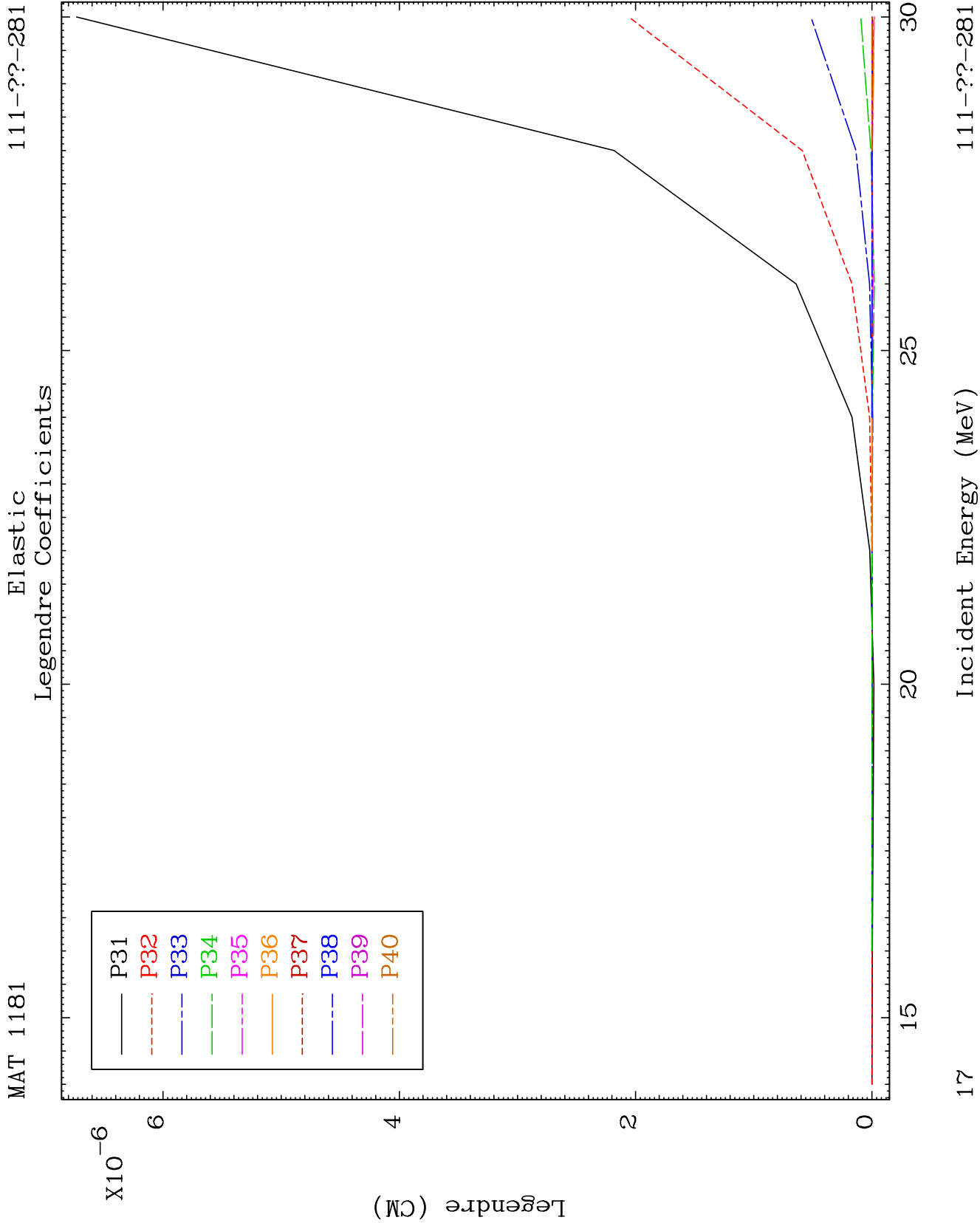
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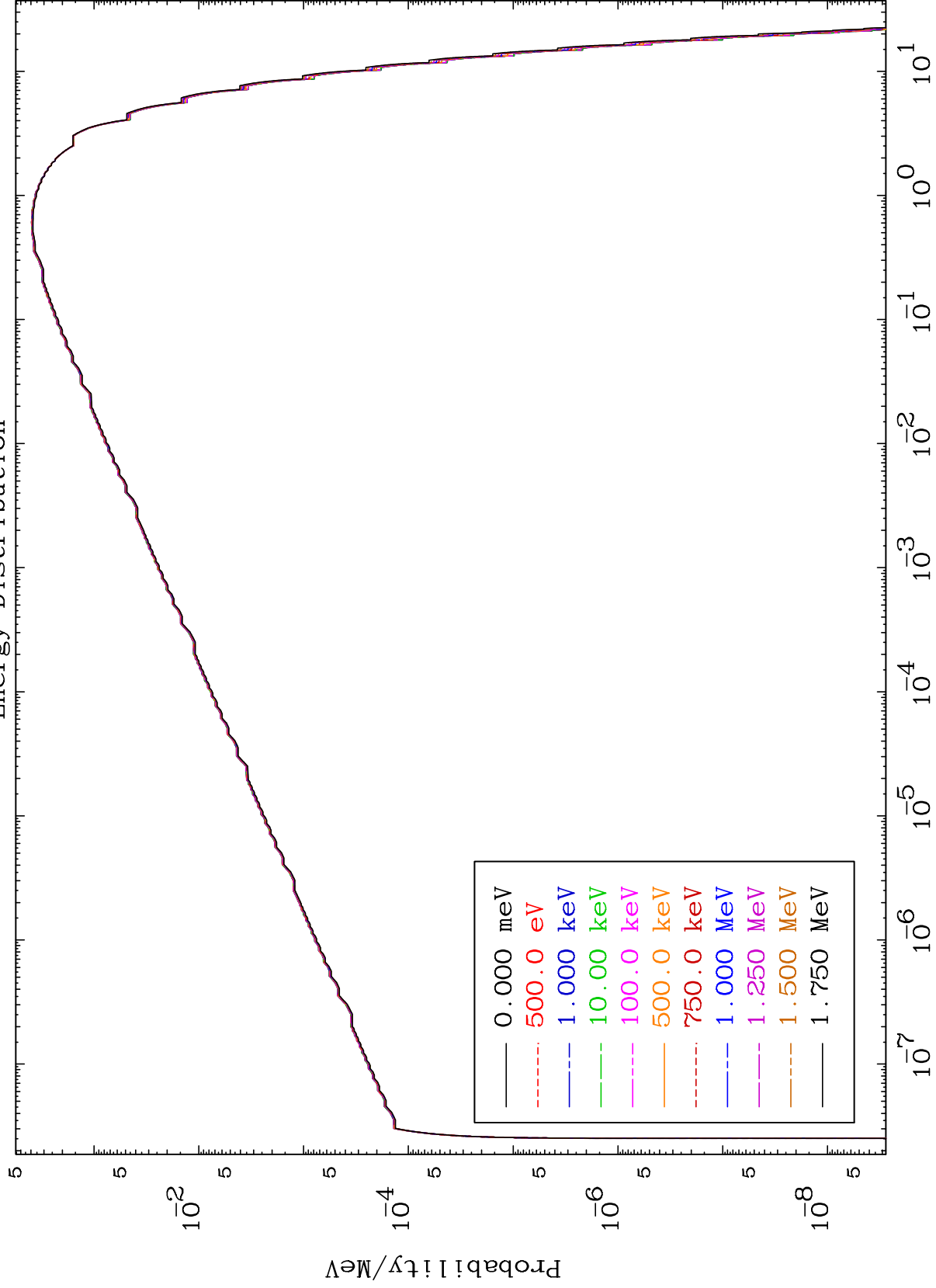


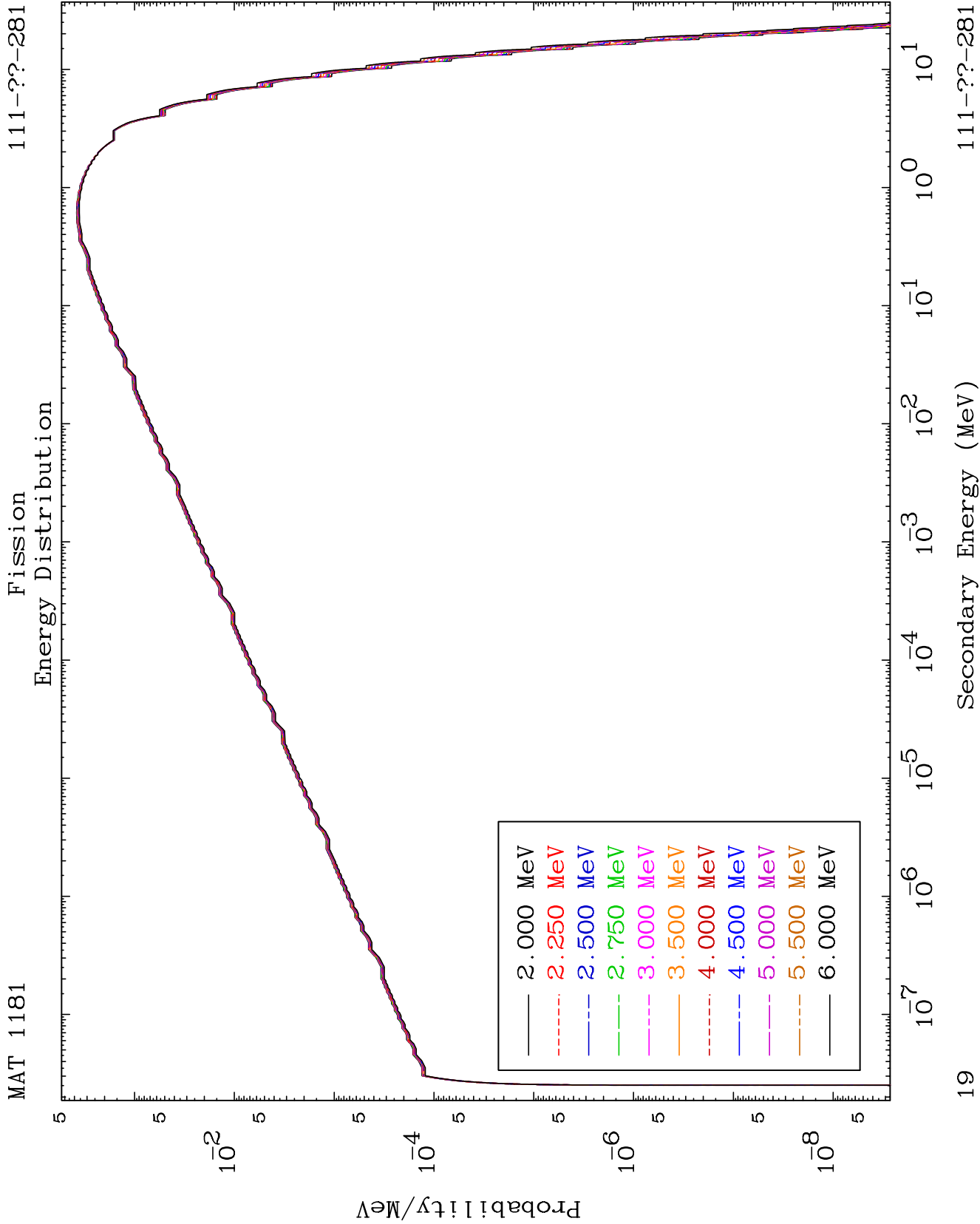


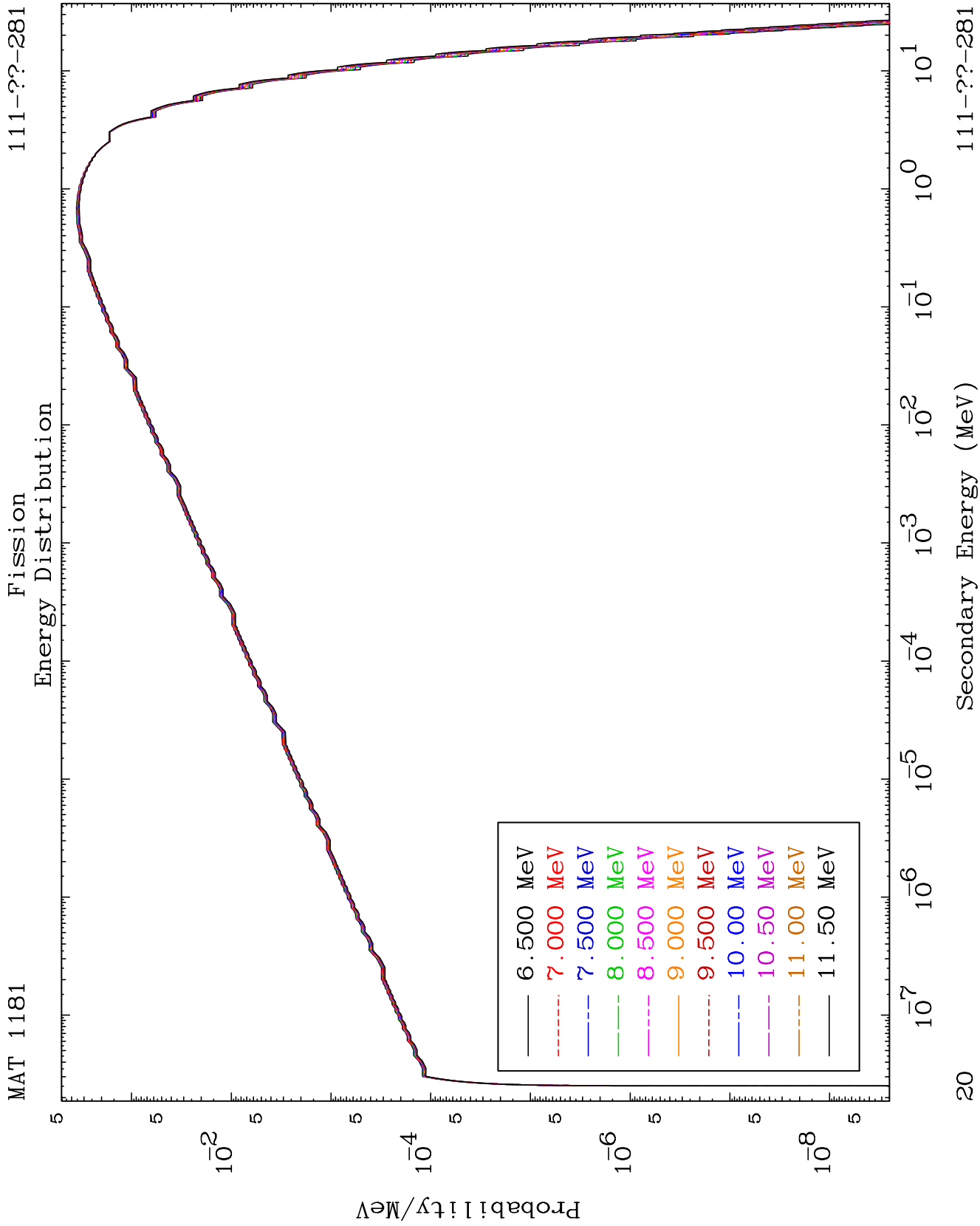
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Fission
Energy Distribution

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Fission
Energy Distribution