

Program EVALPLOT
(Version 2017-1)

by

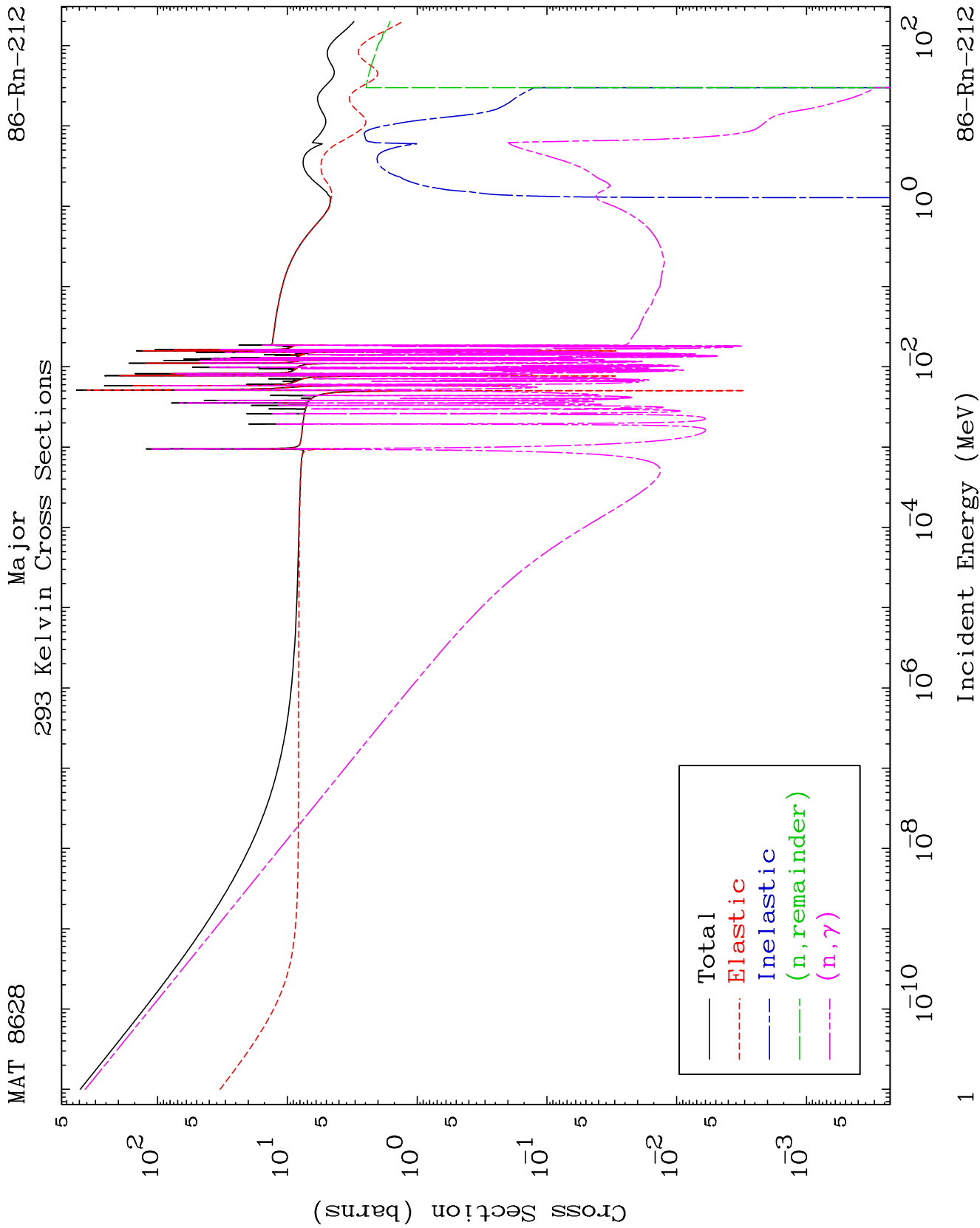
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

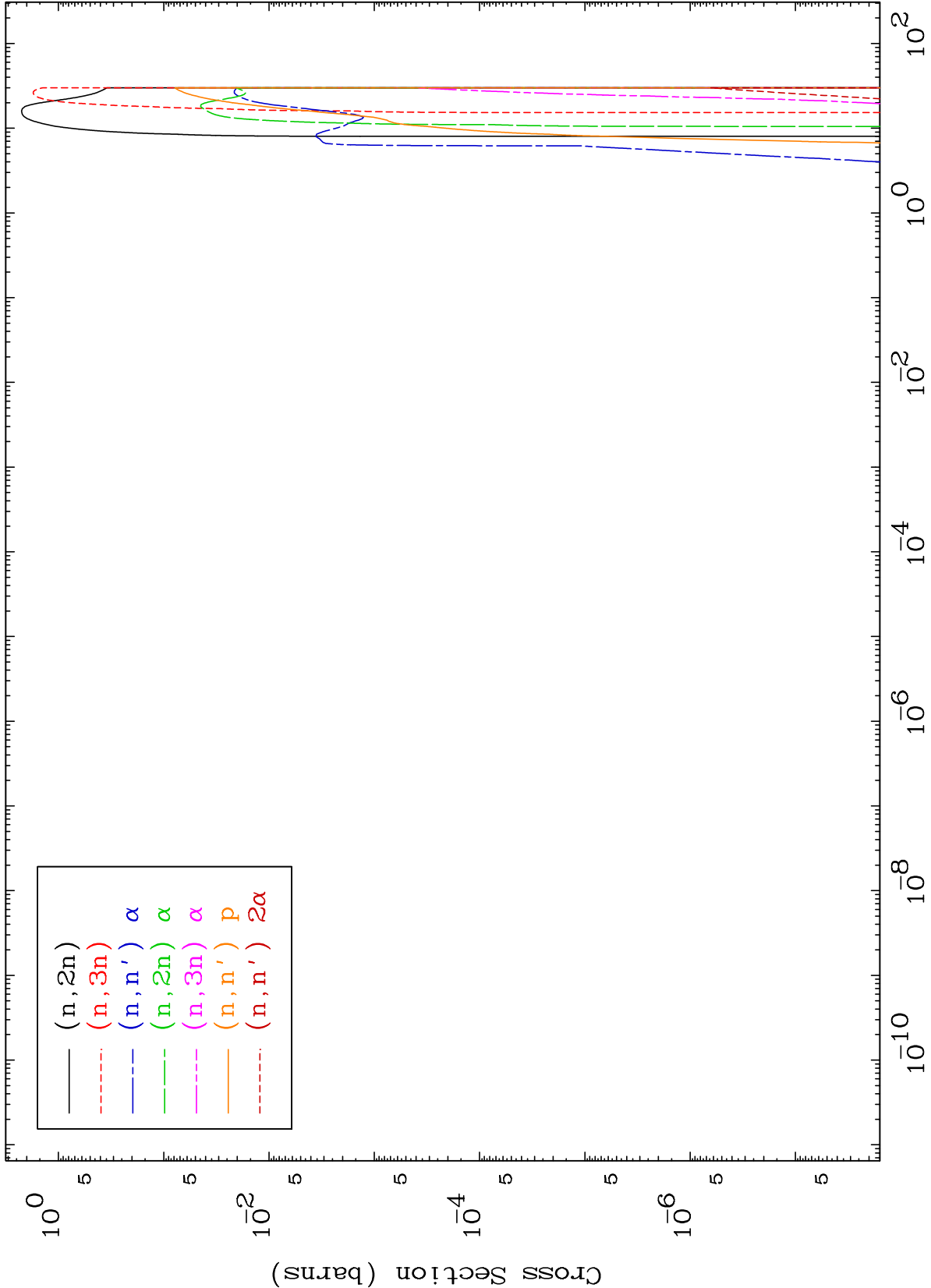
Press Mouse Button to Start

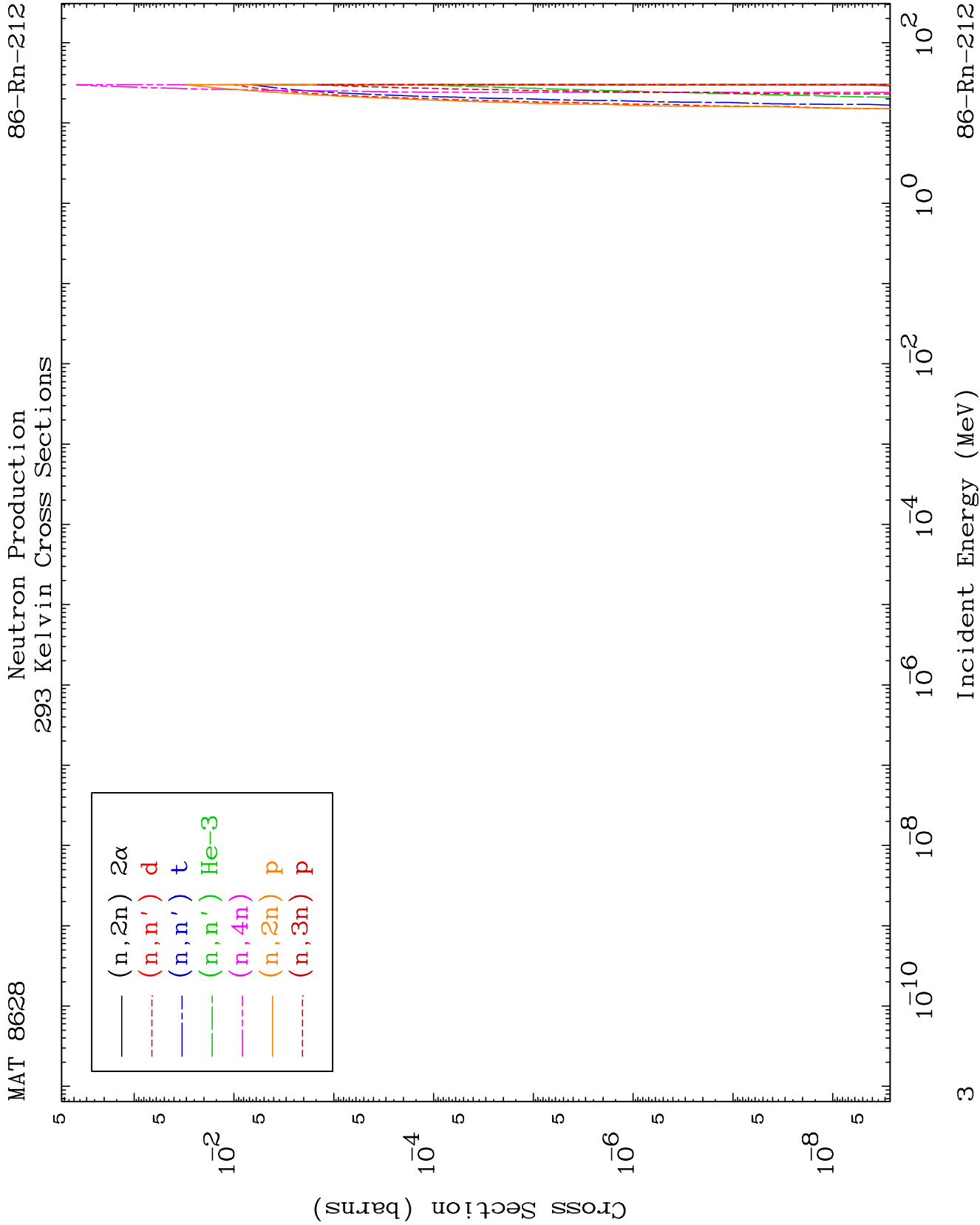


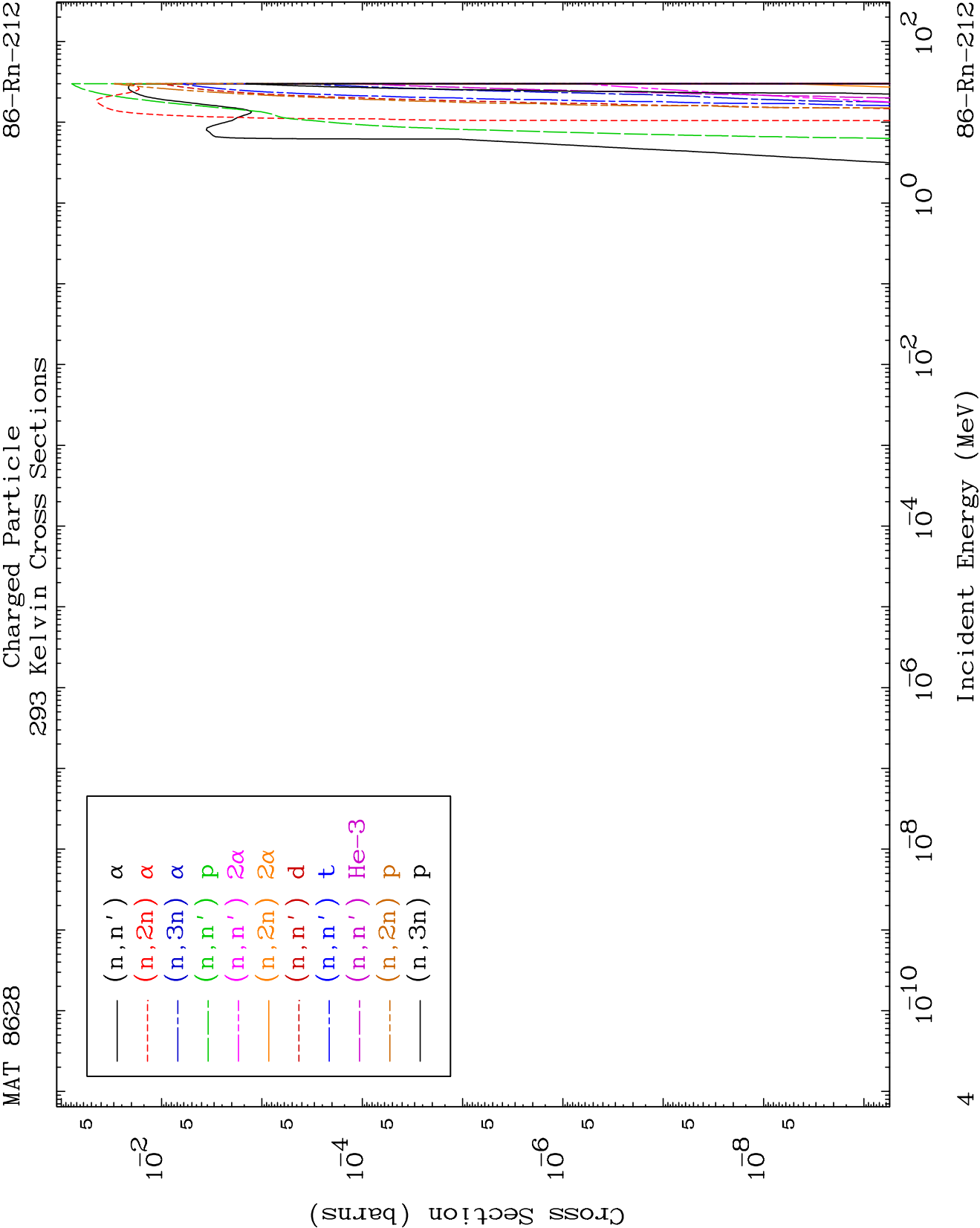
MAT 8628

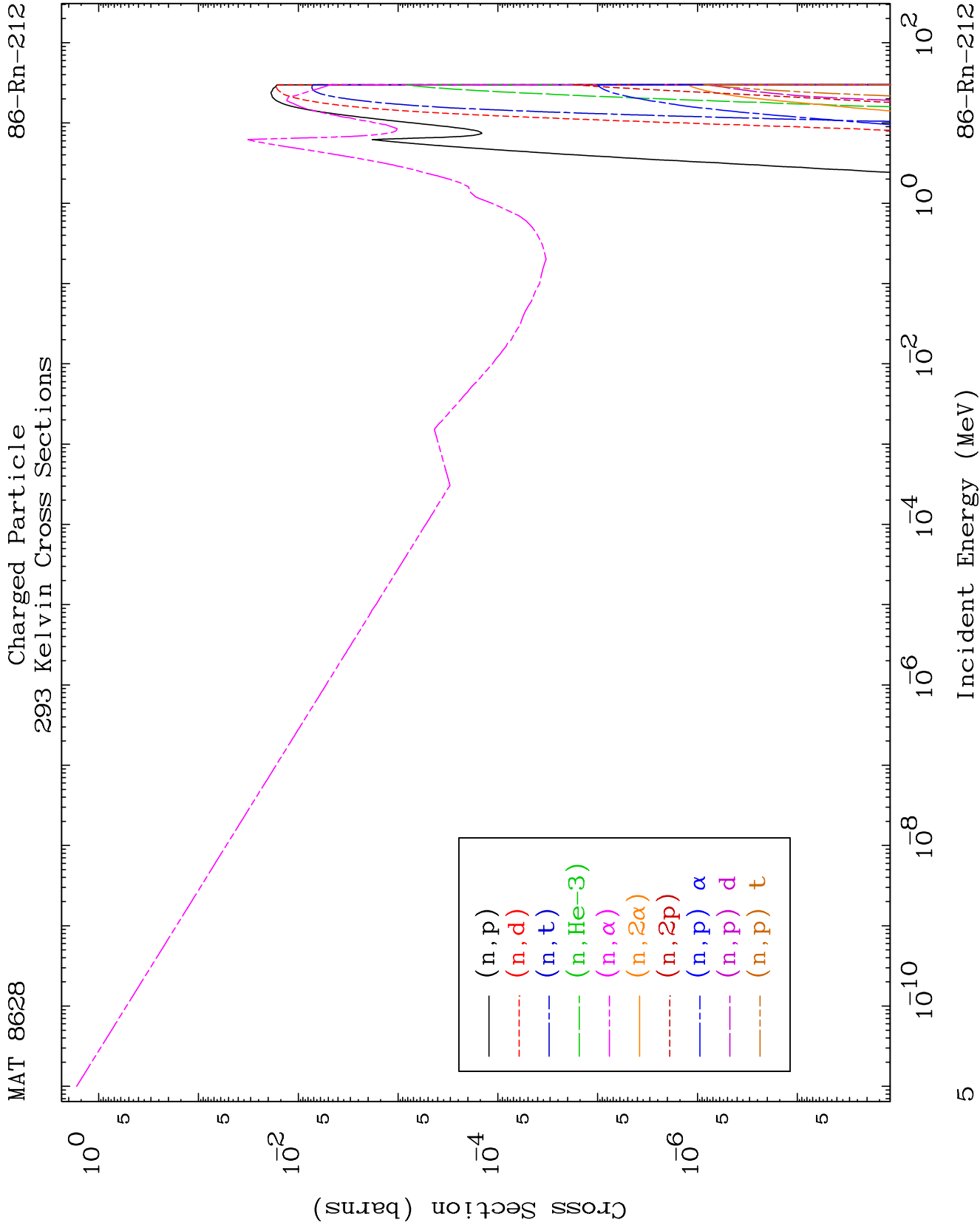
Neutron Production
293 Kelvin Cross Sections

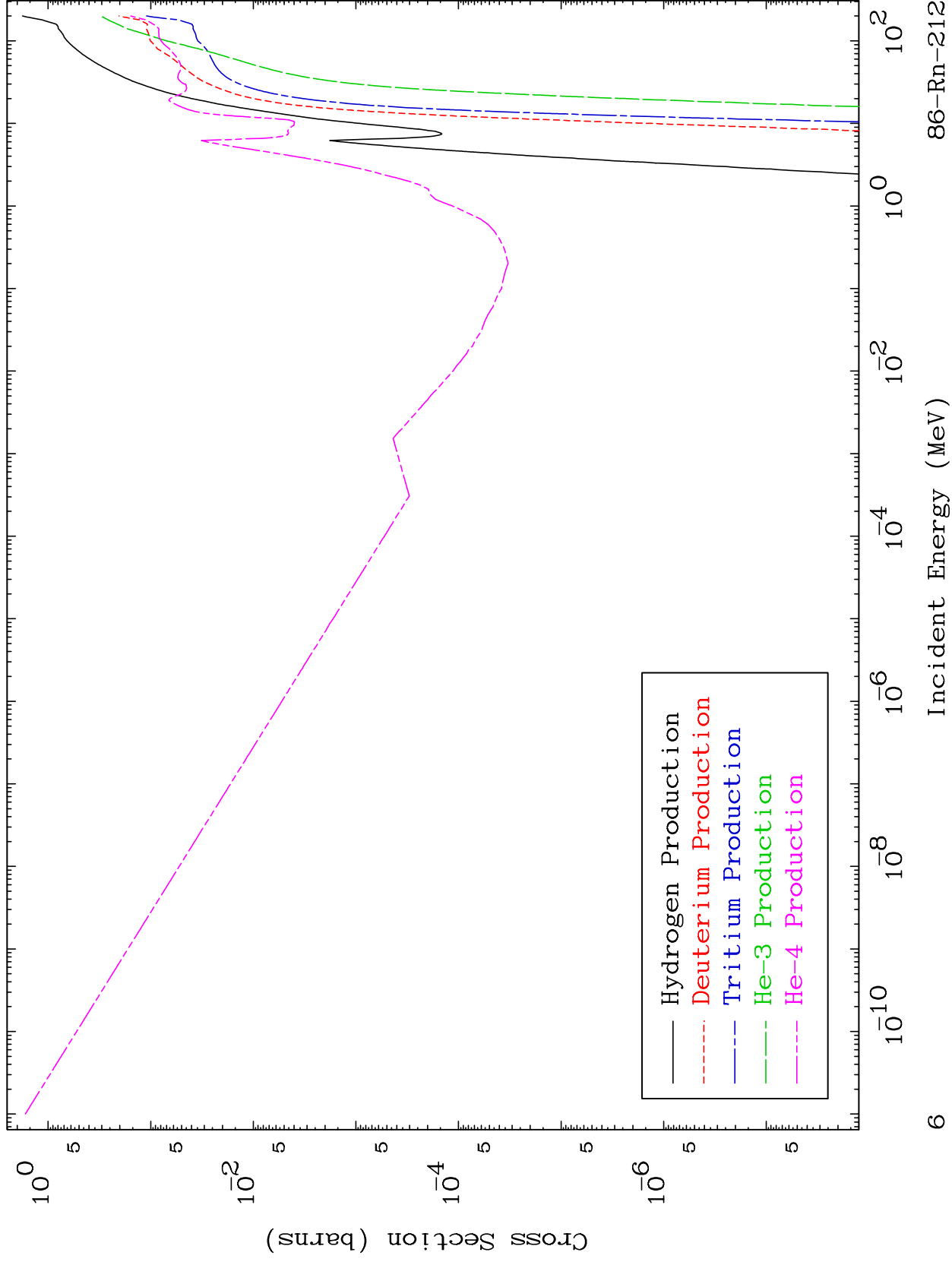
86-Rn-212

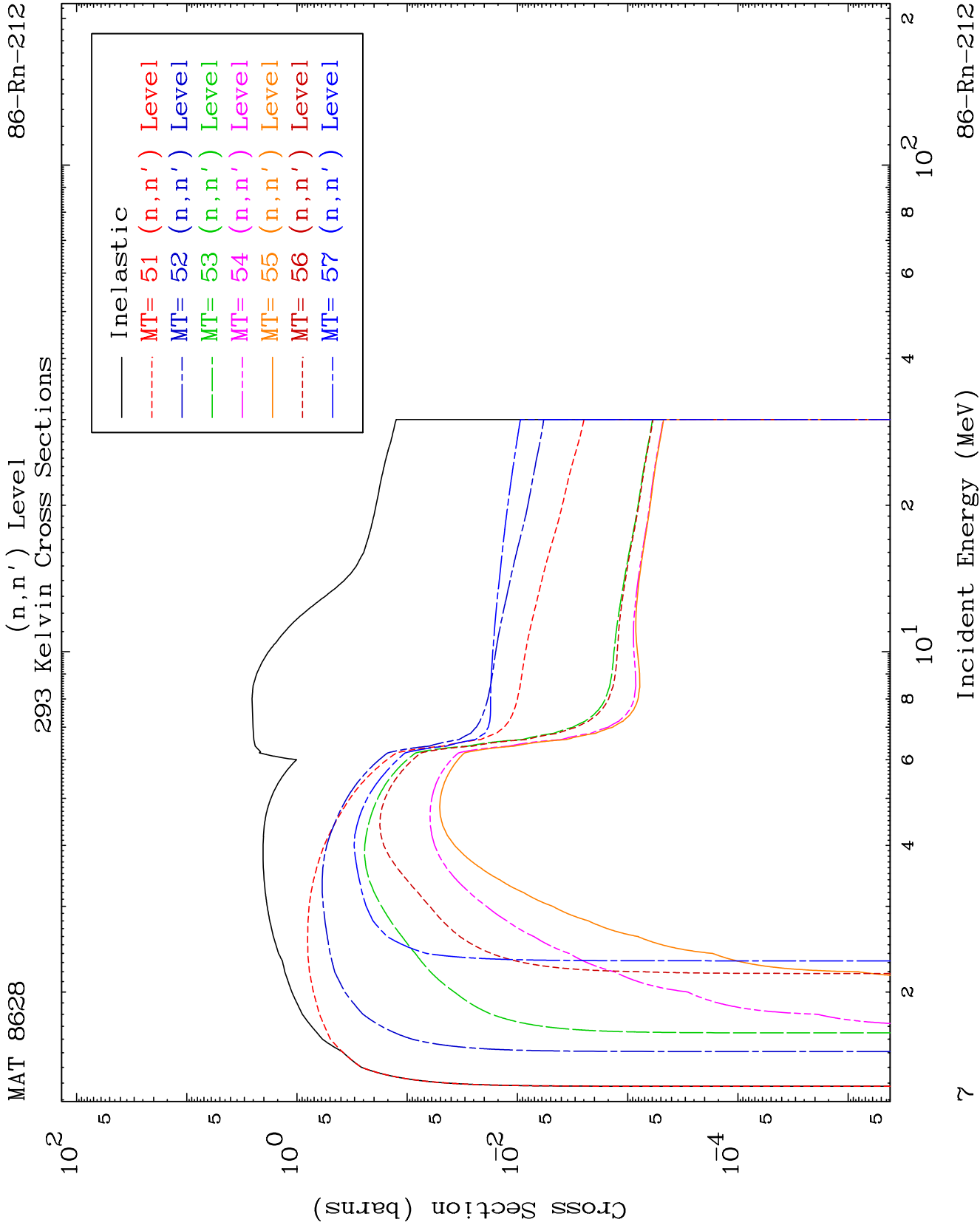


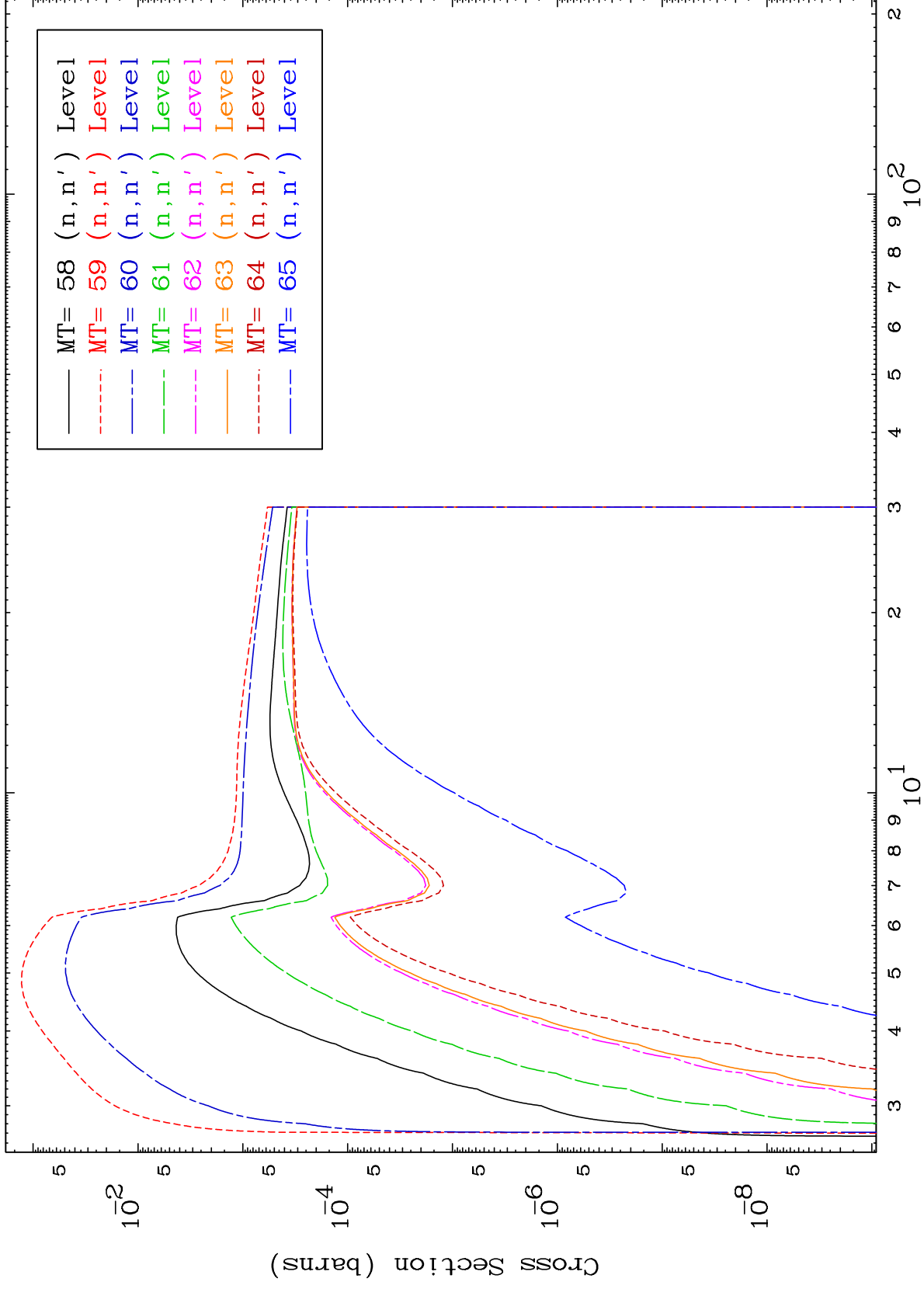


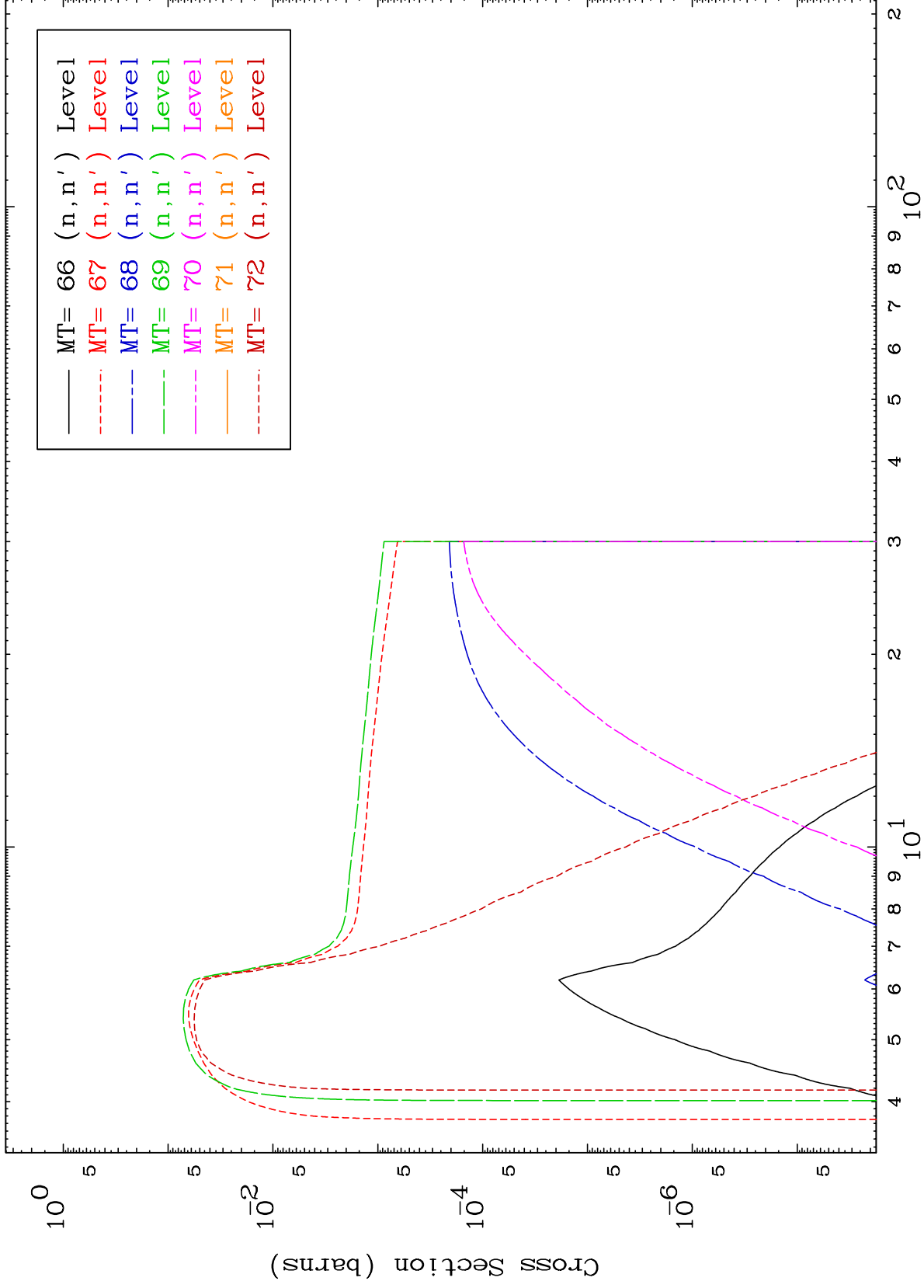








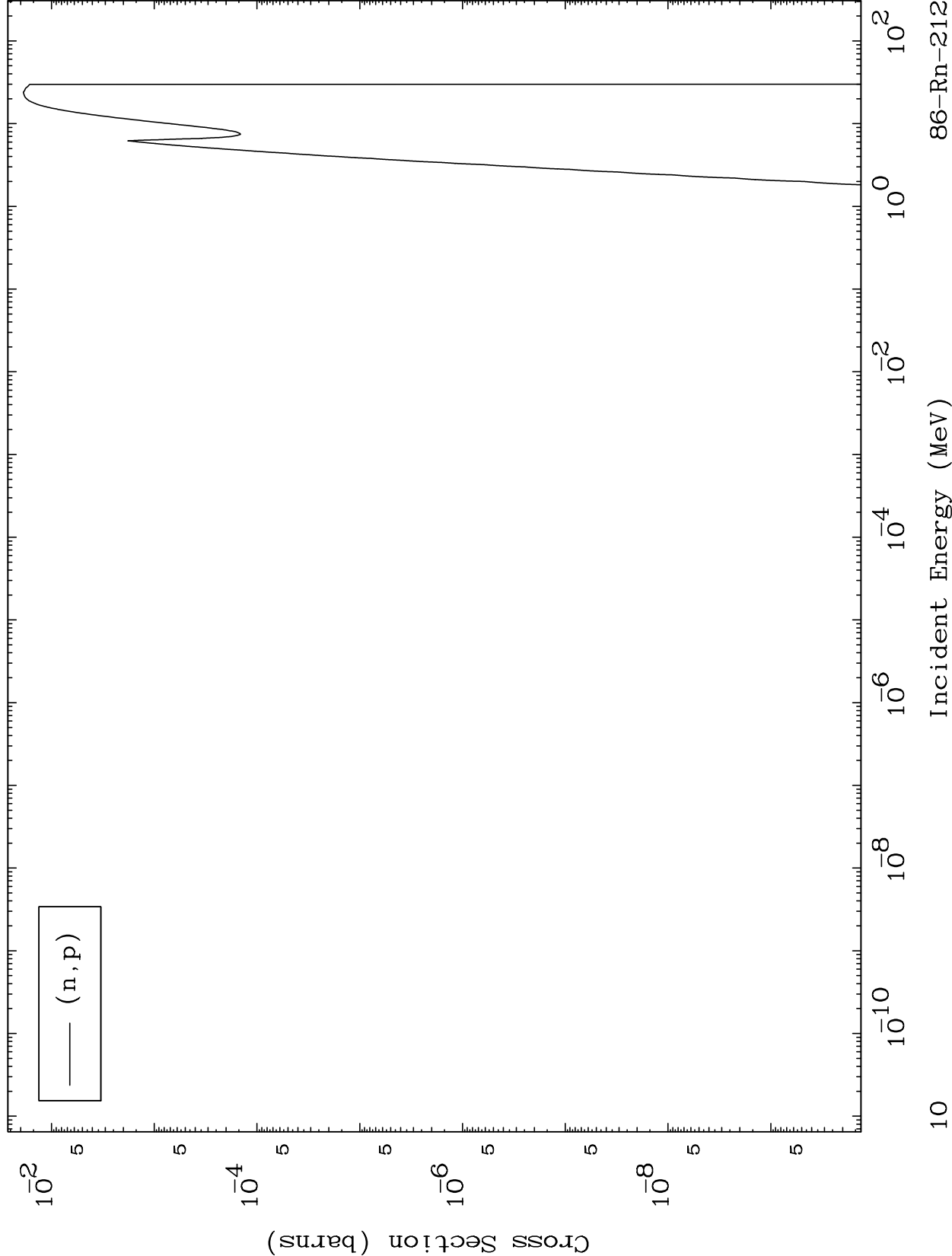




MAT 8628

(n,p) Levels
293 Kelvin Cross Sections

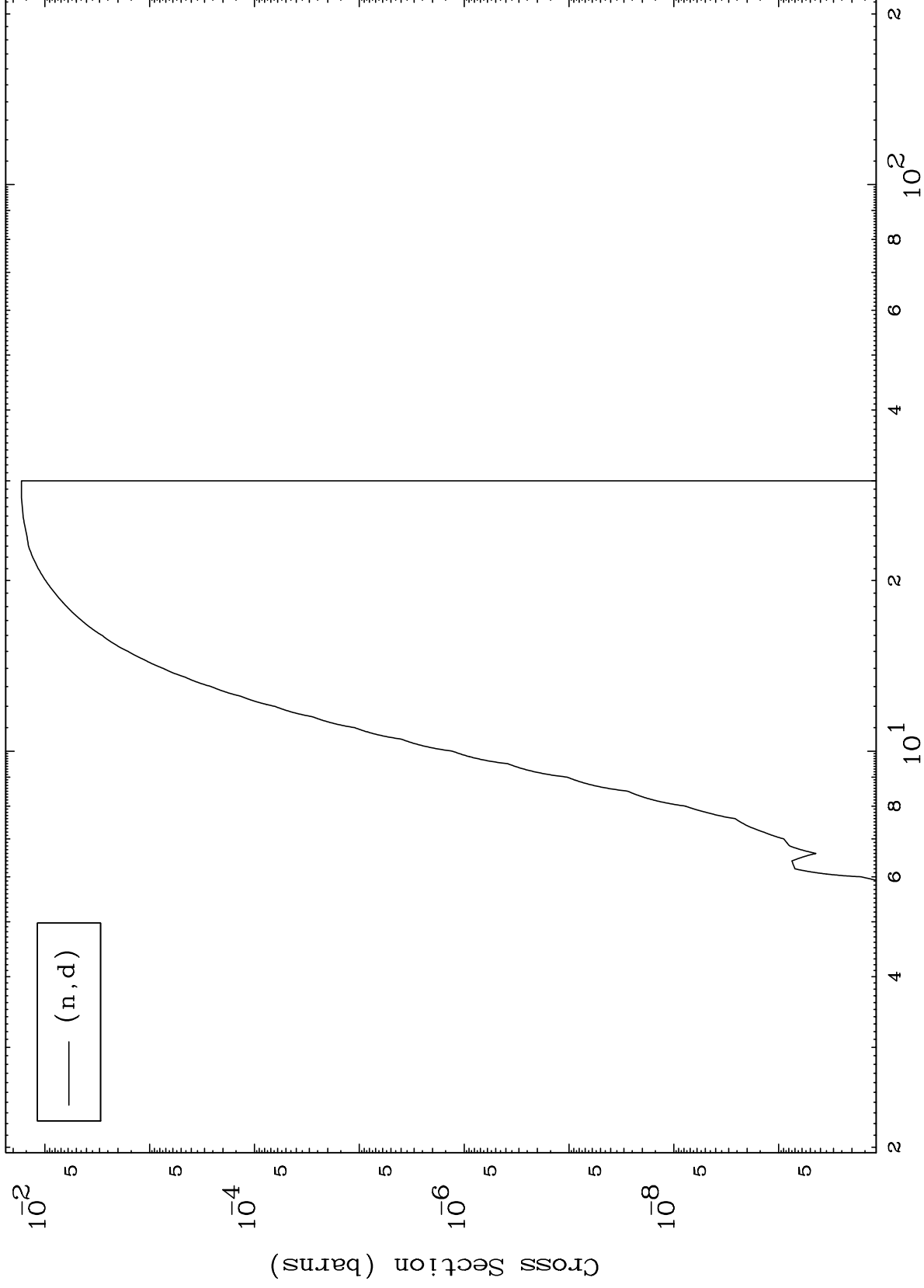
86-Rn-212



MAT 8628

86-Rn-212

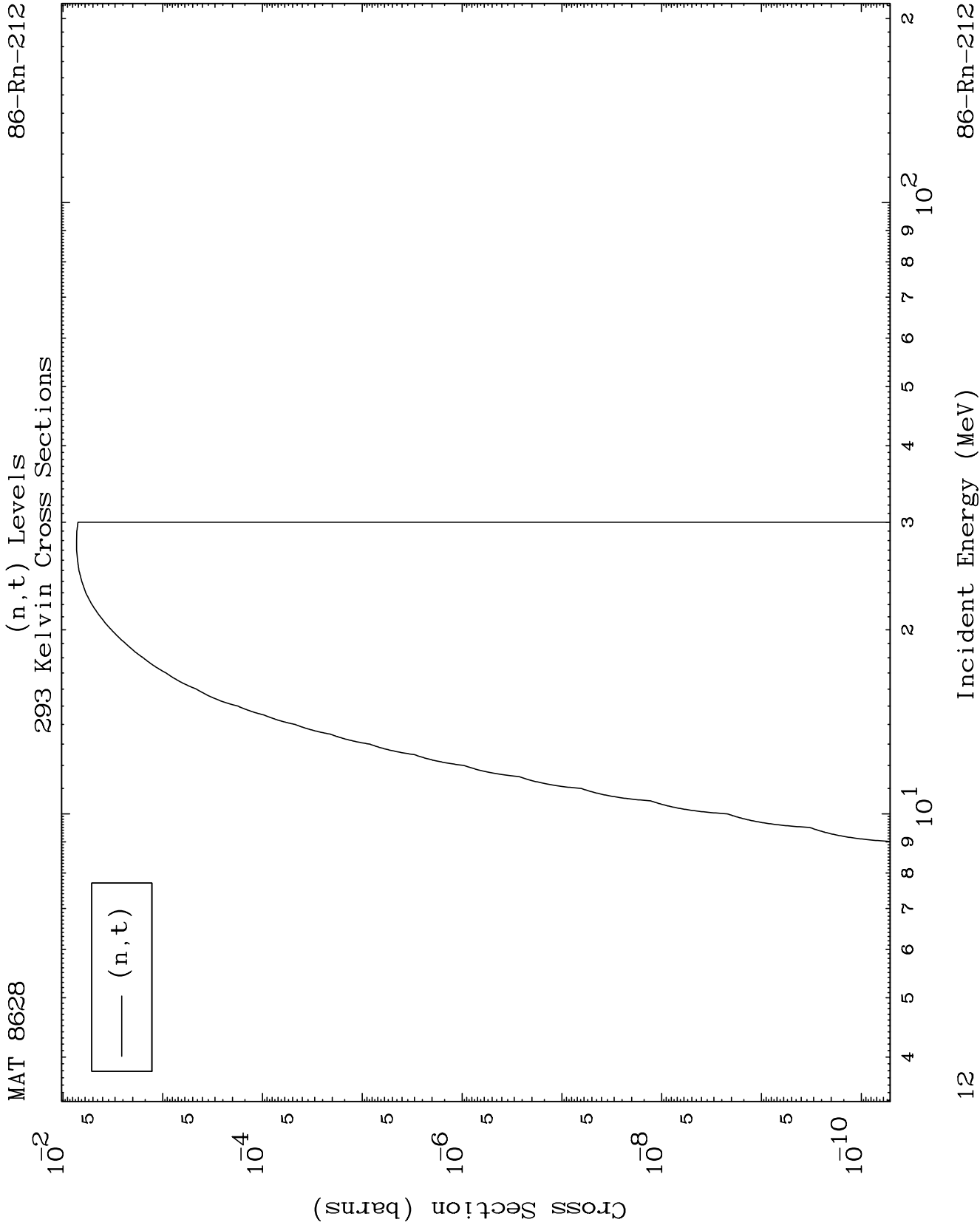
(n,d) Levels
293 Kelvin Cross Sections

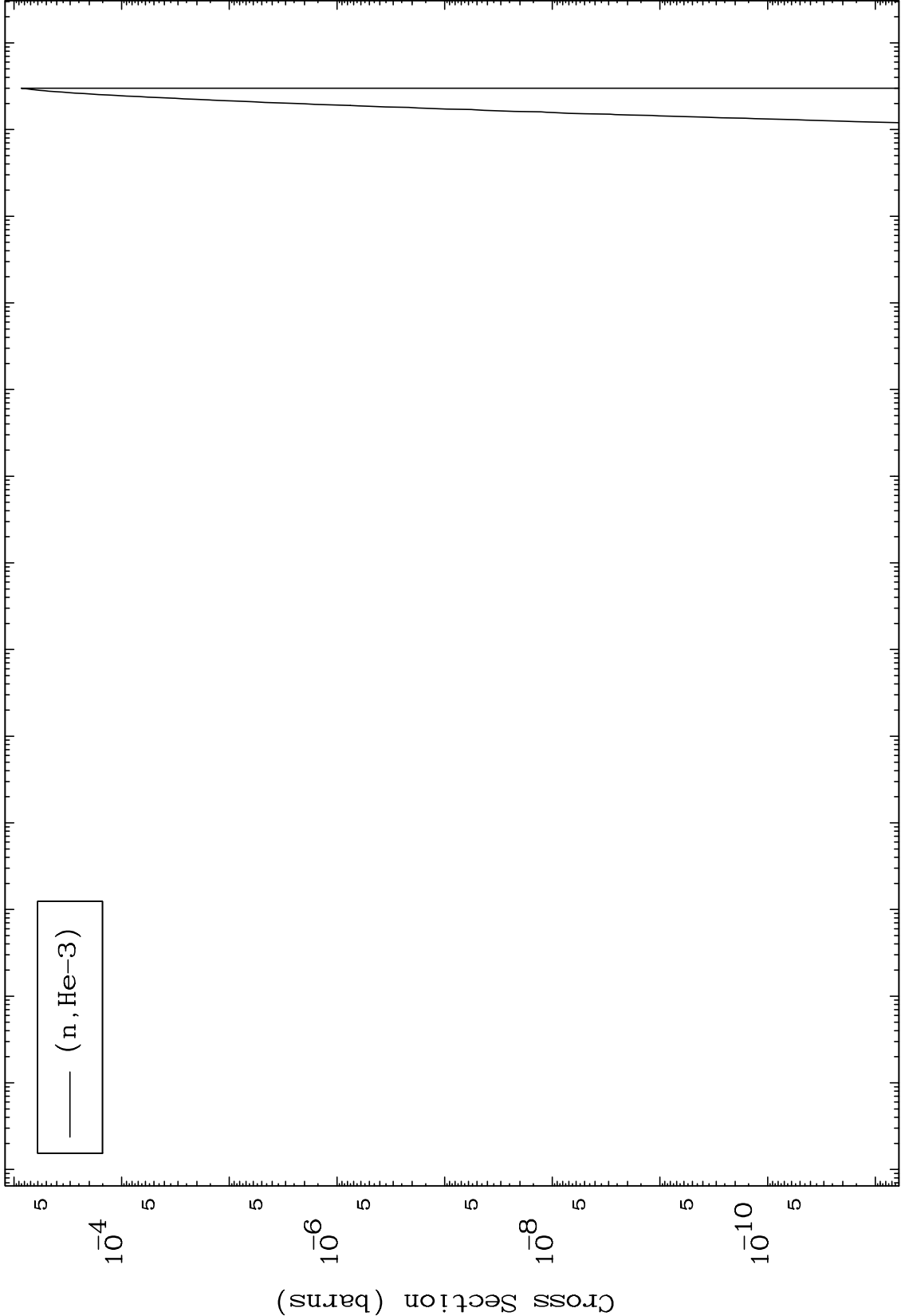


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86-Rn-212

Incident Energy (MeV)

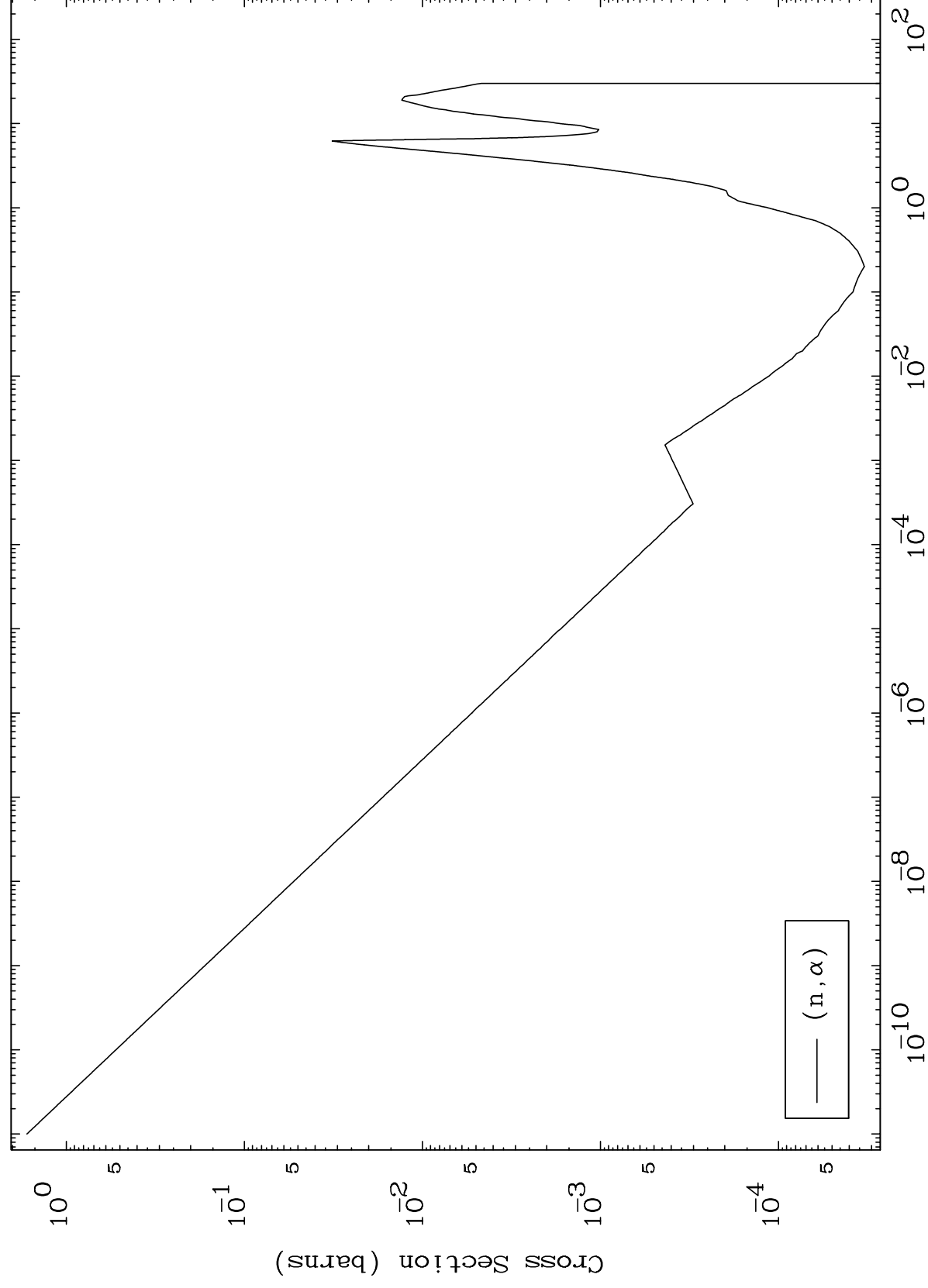




MAT 8628

(n, α) Levels
293 Kelvin Cross Sections

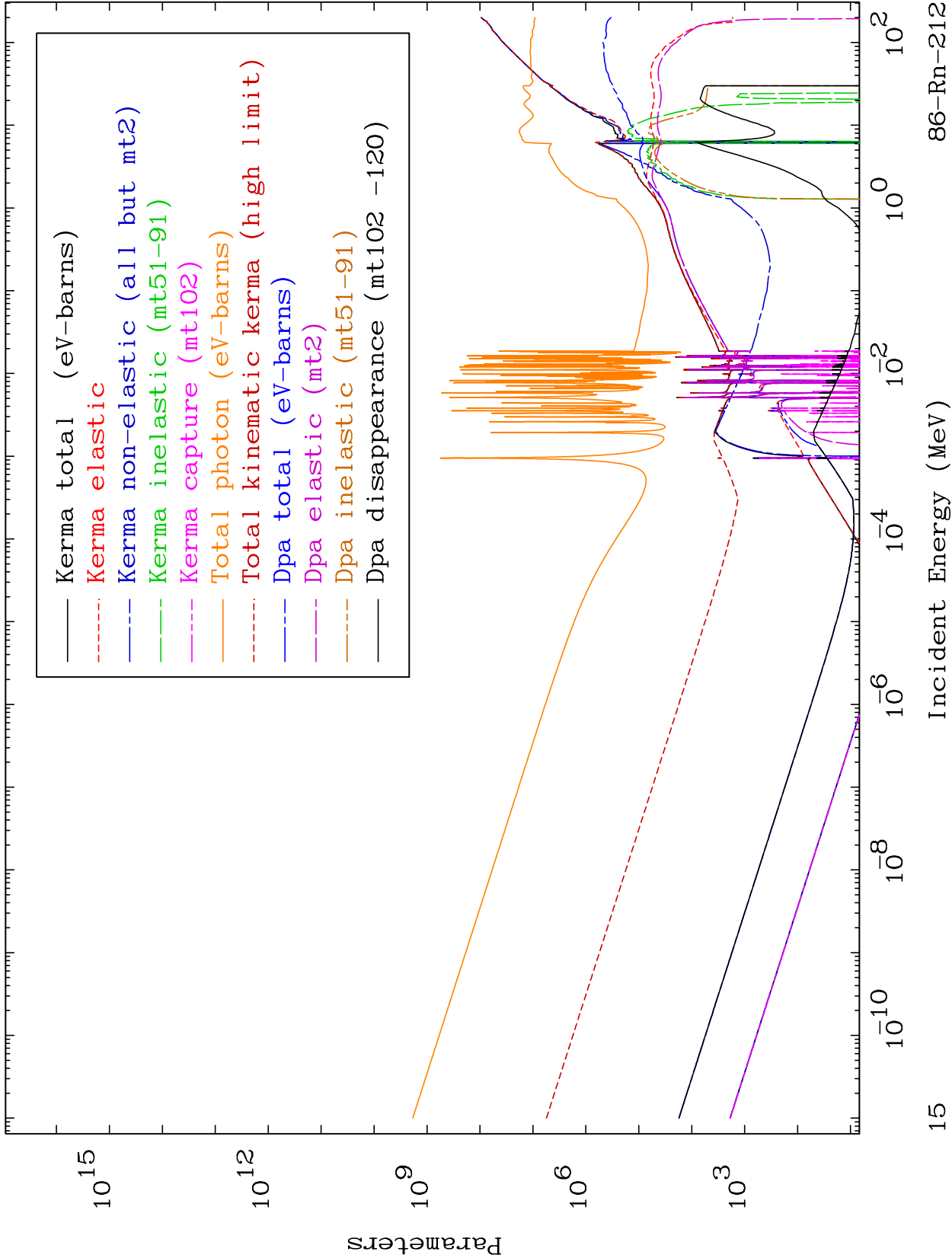
86-Rn-212

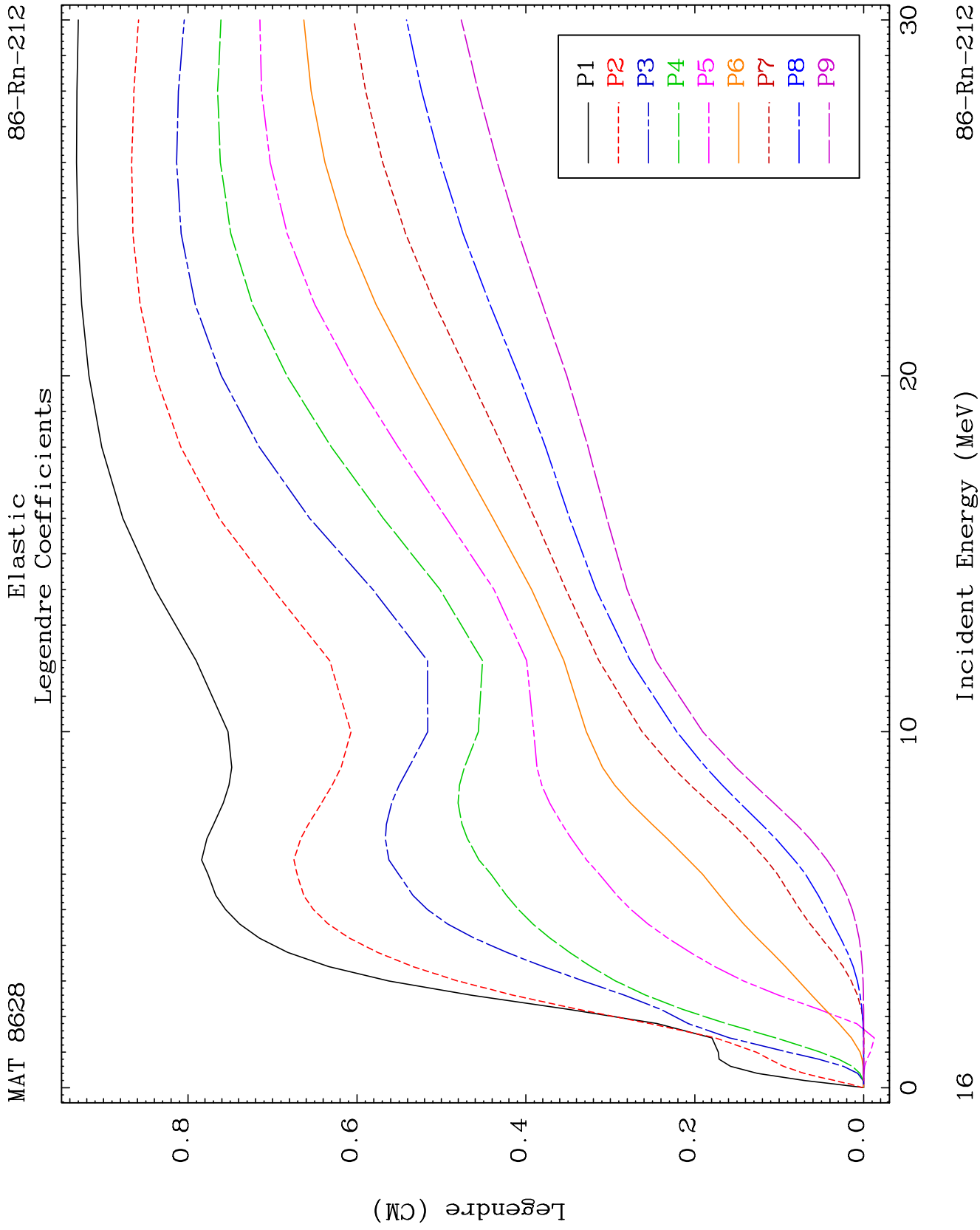


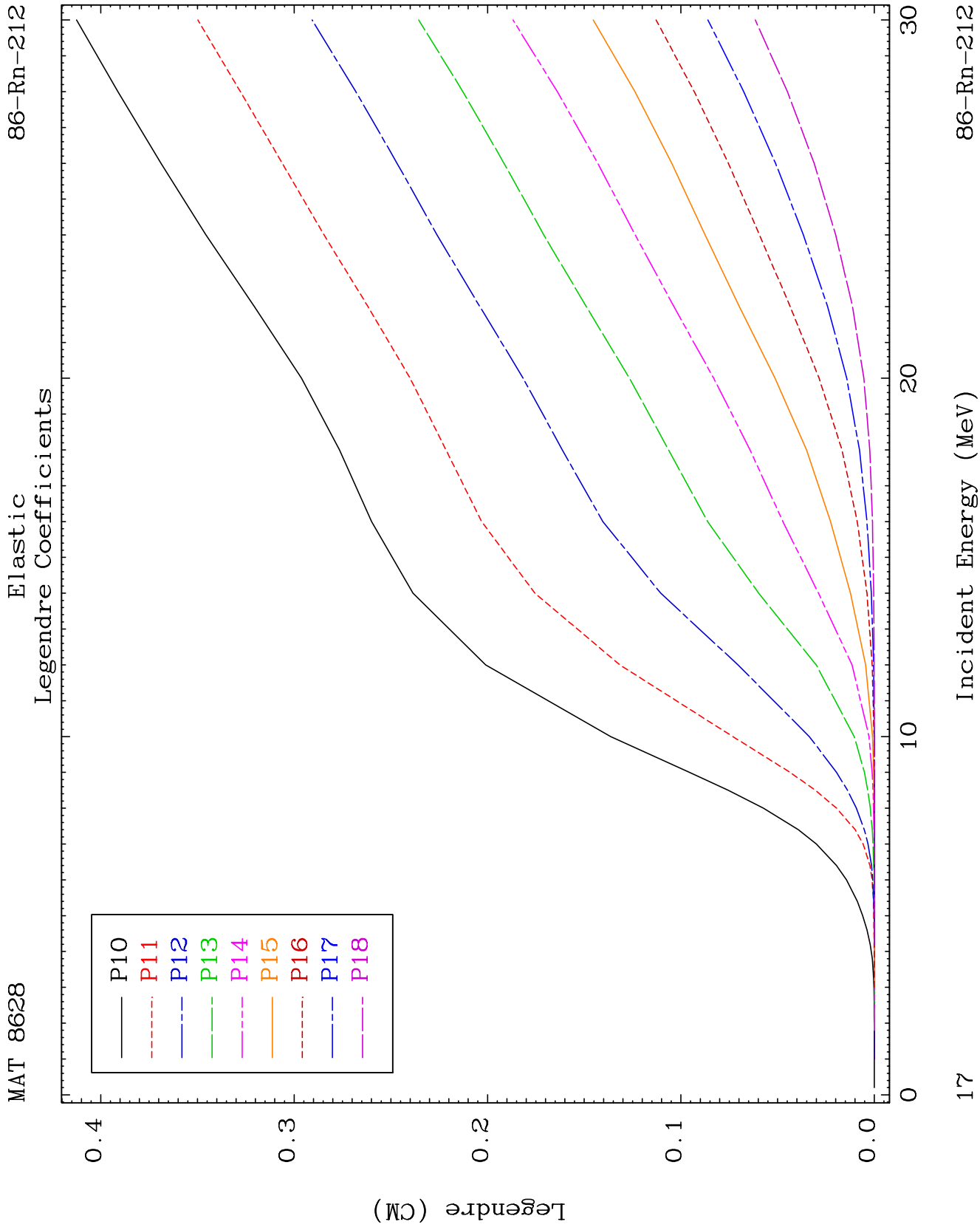
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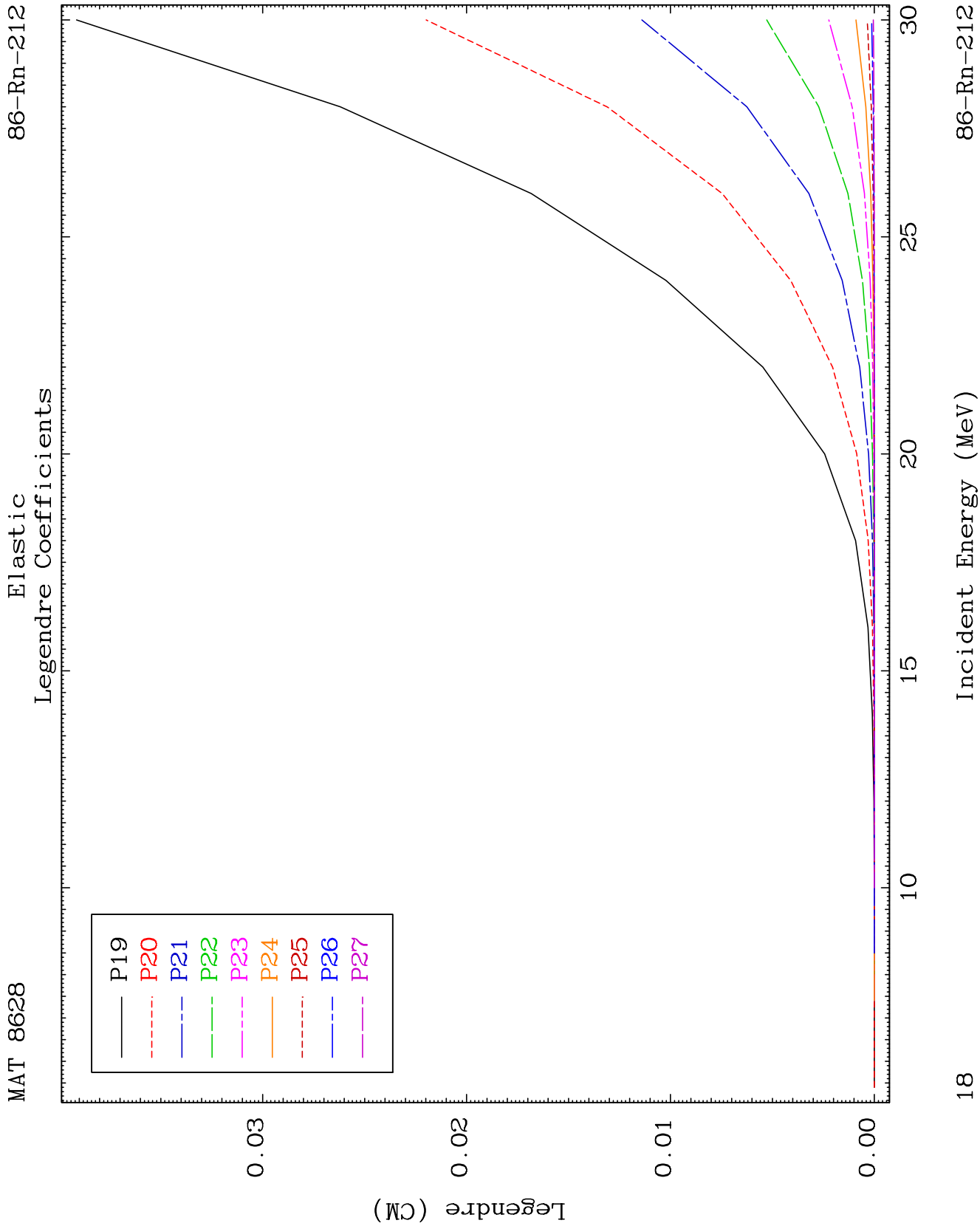
Incident Energy (MeV)

86-Rn-212





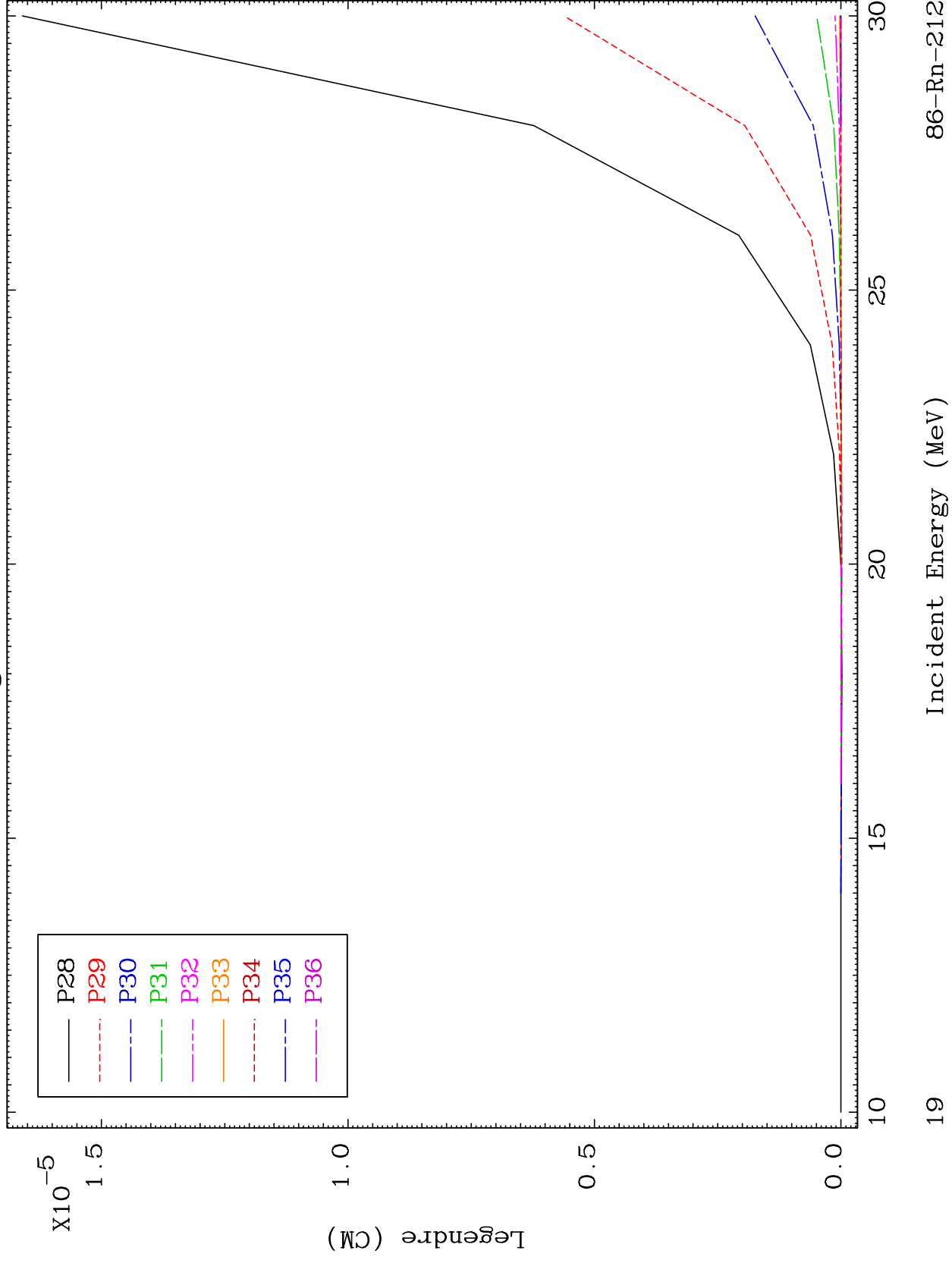




MAT 8628

Elastic Legendre Coefficients

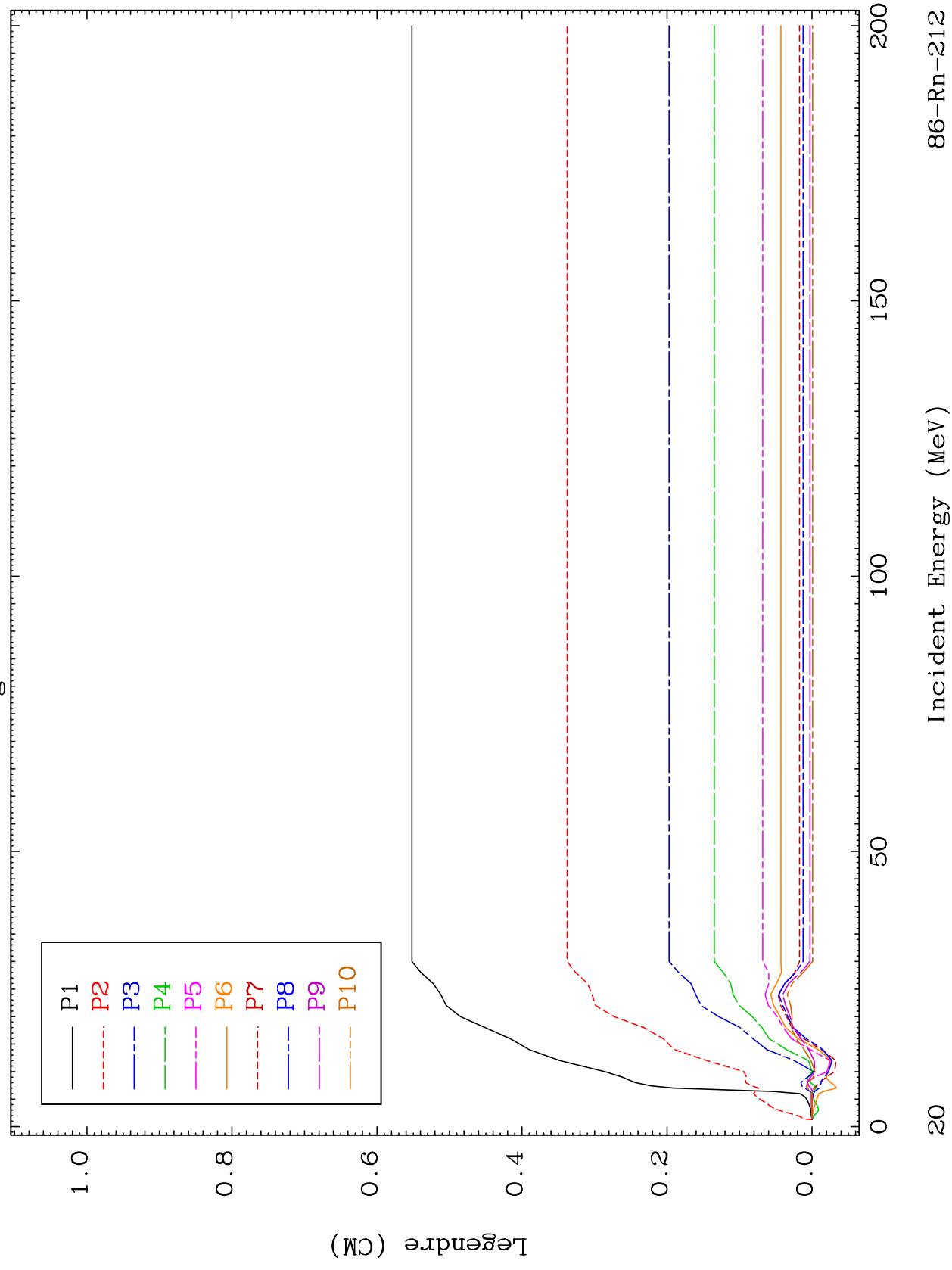
86-Rn-212



MAT 8628

MT= 51 (n, n') Level
Legendre Coefficients

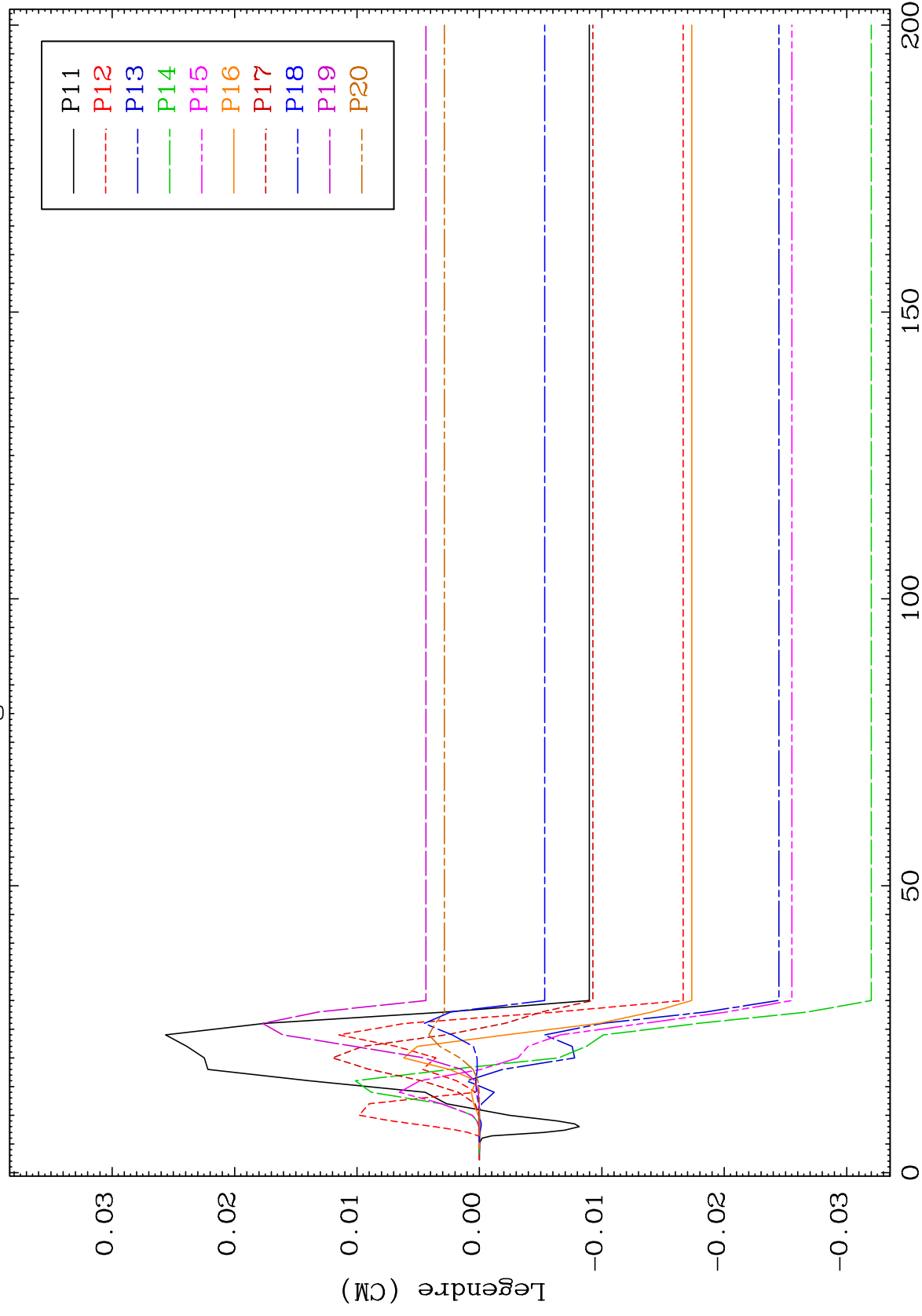
86-Rn-212



MAT 8628

MT= 51 (n, n') Level
Legendre Coefficients

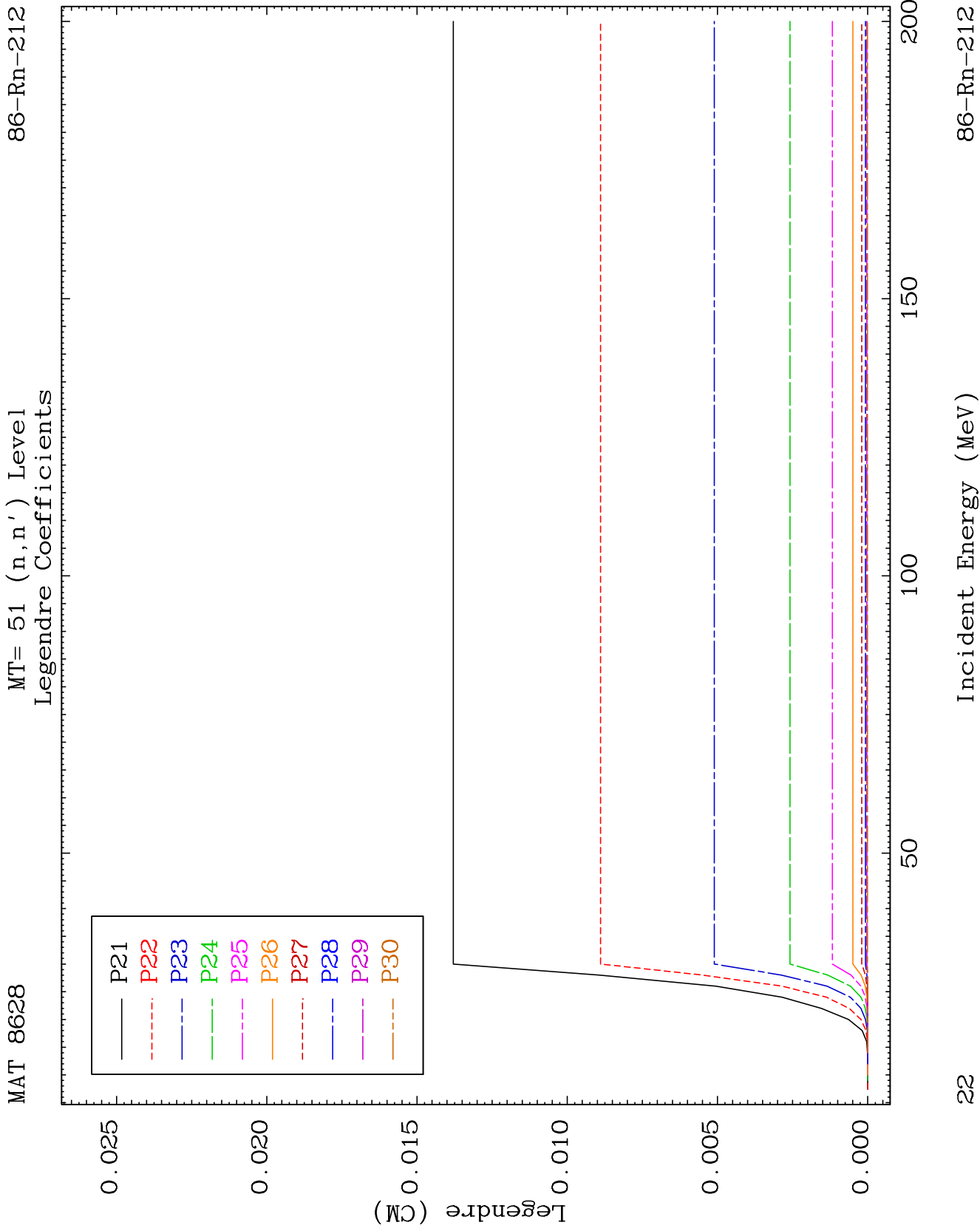
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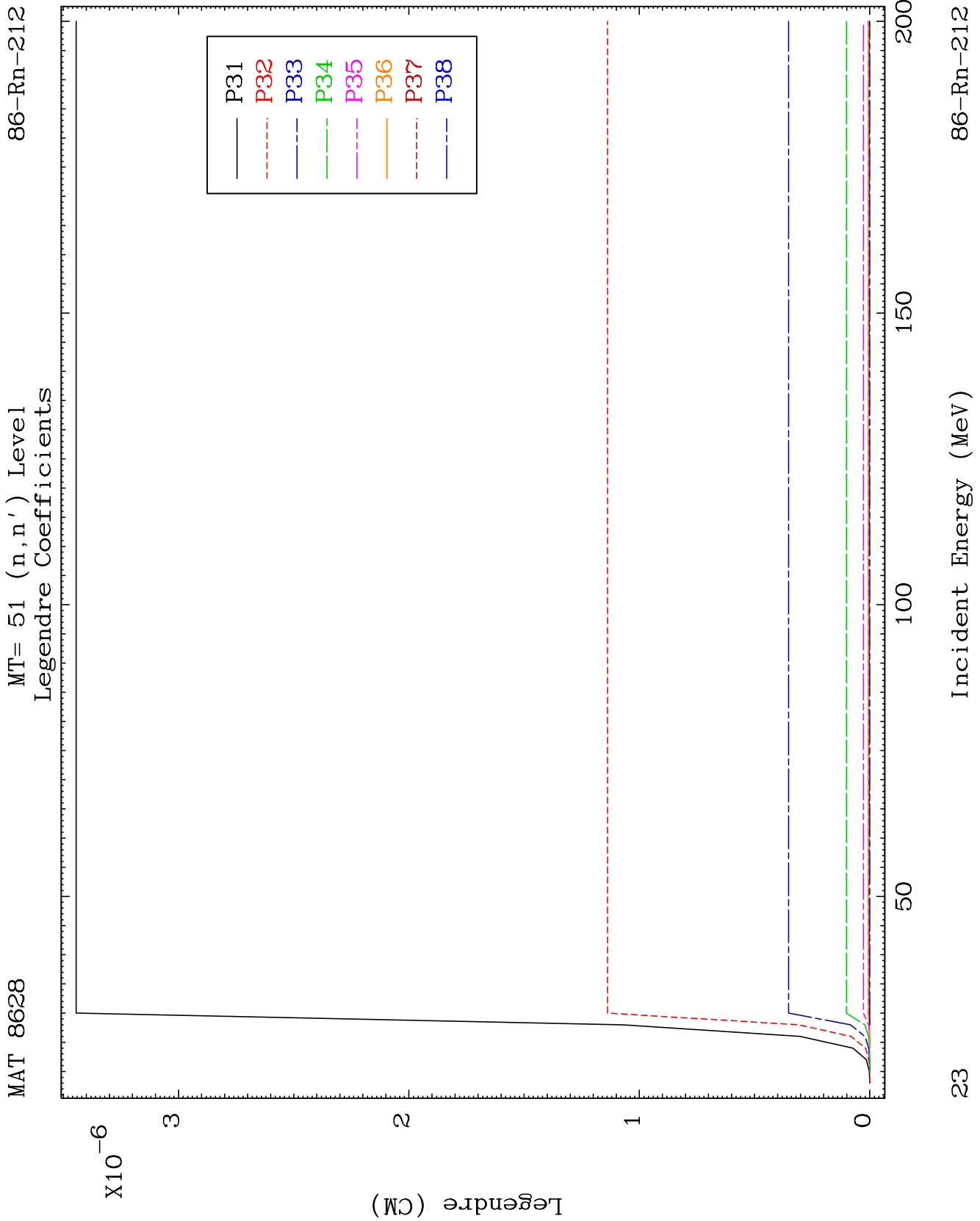


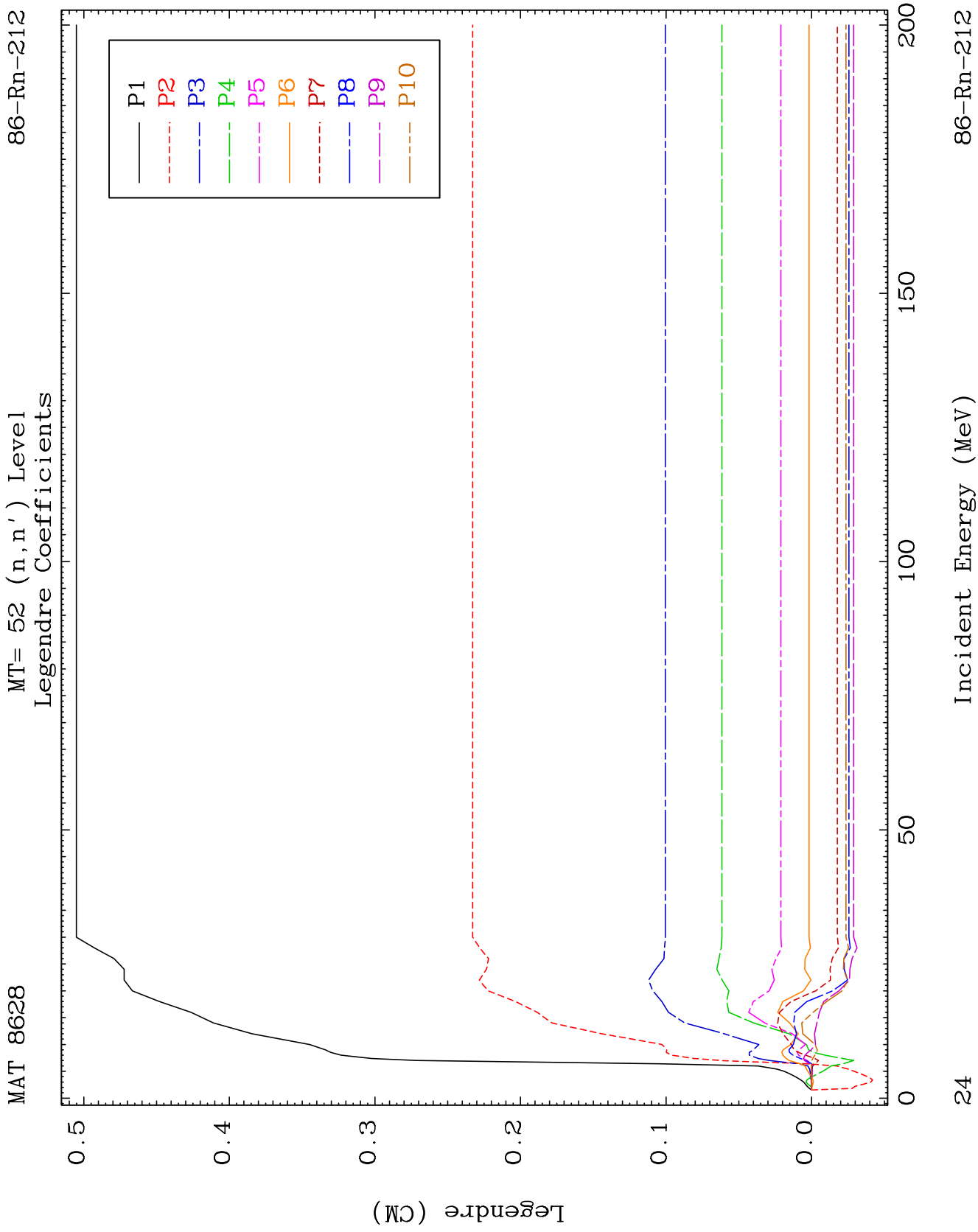
21

Incident Energy (MeV)

86-Rn-212



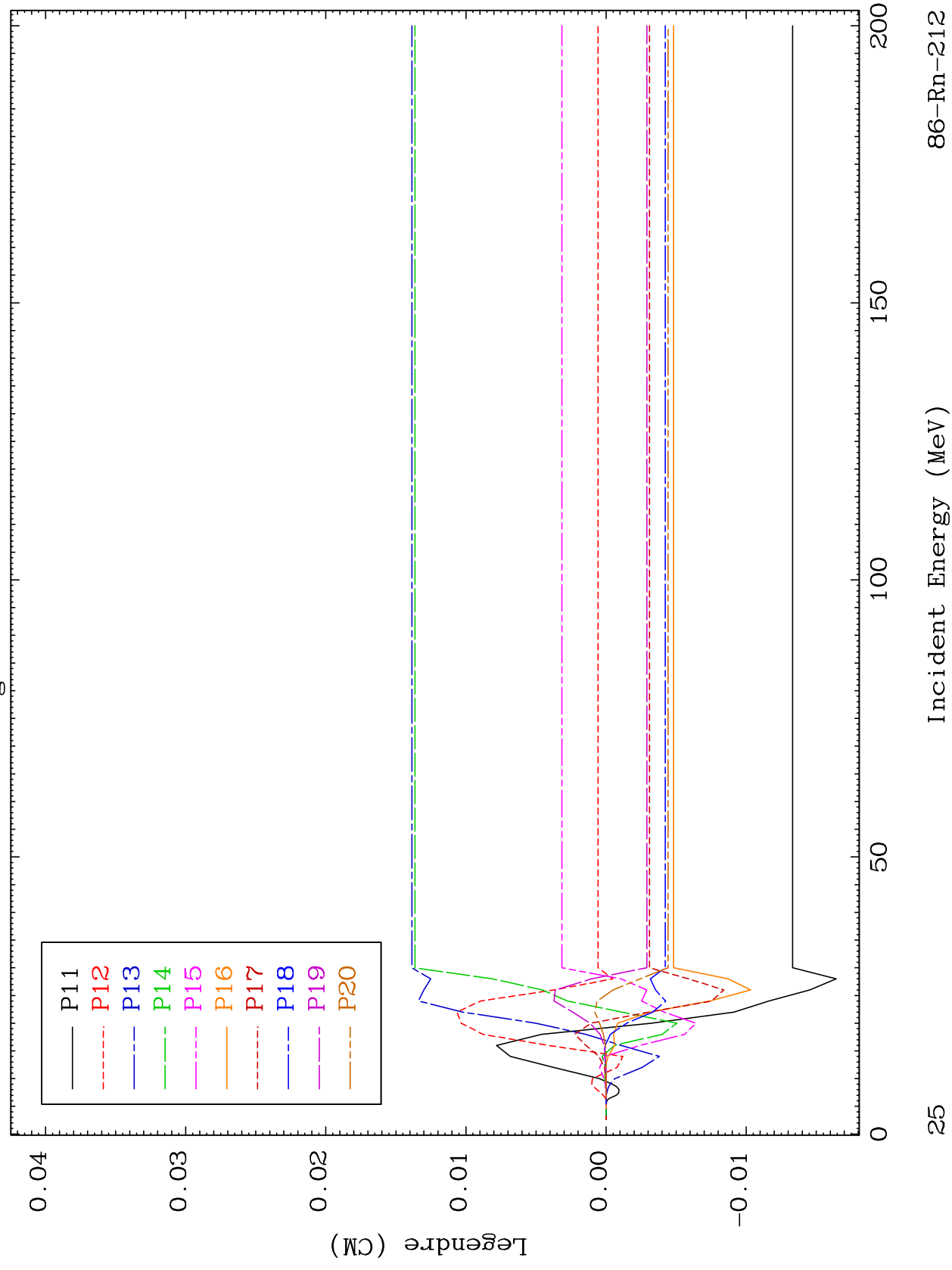


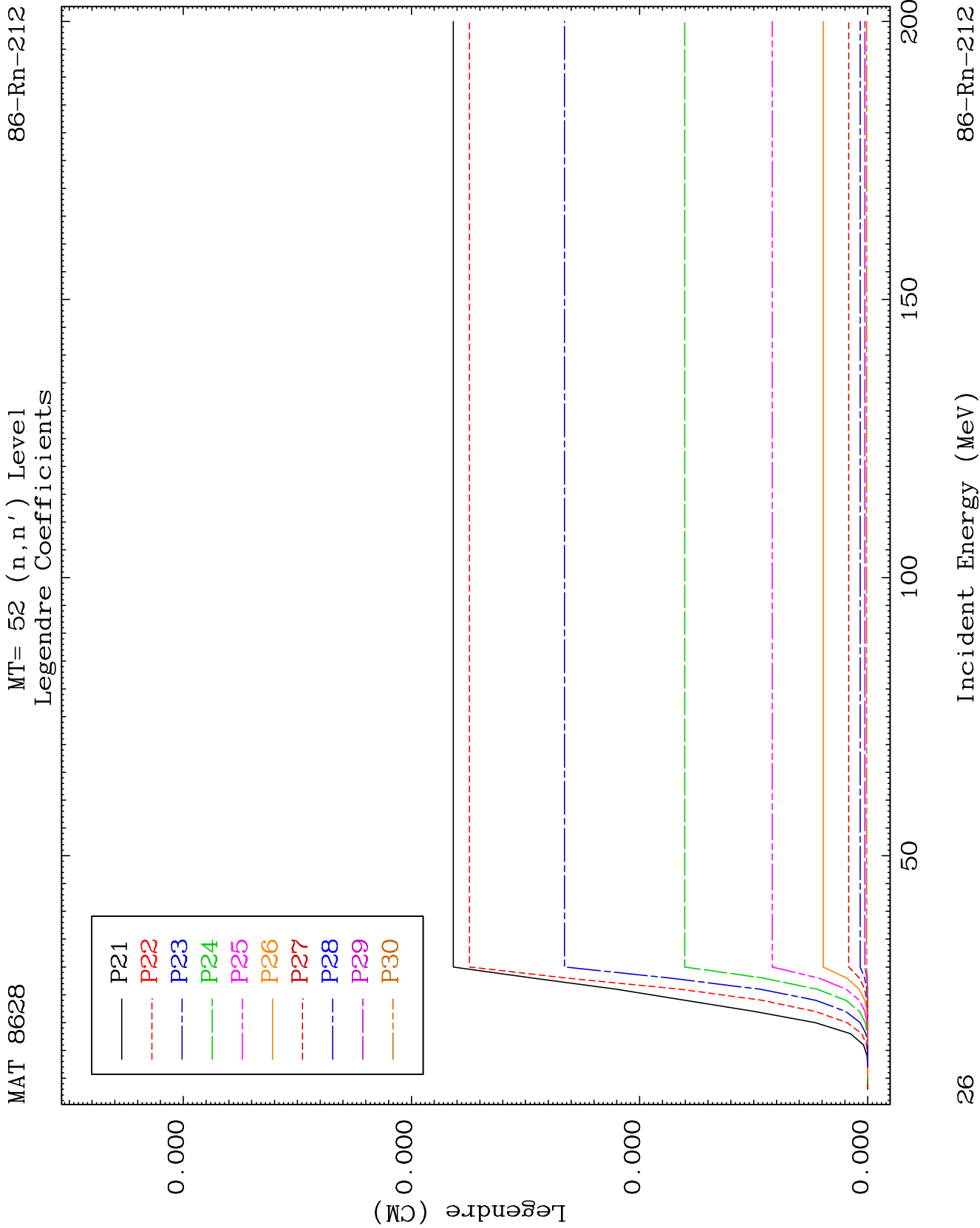


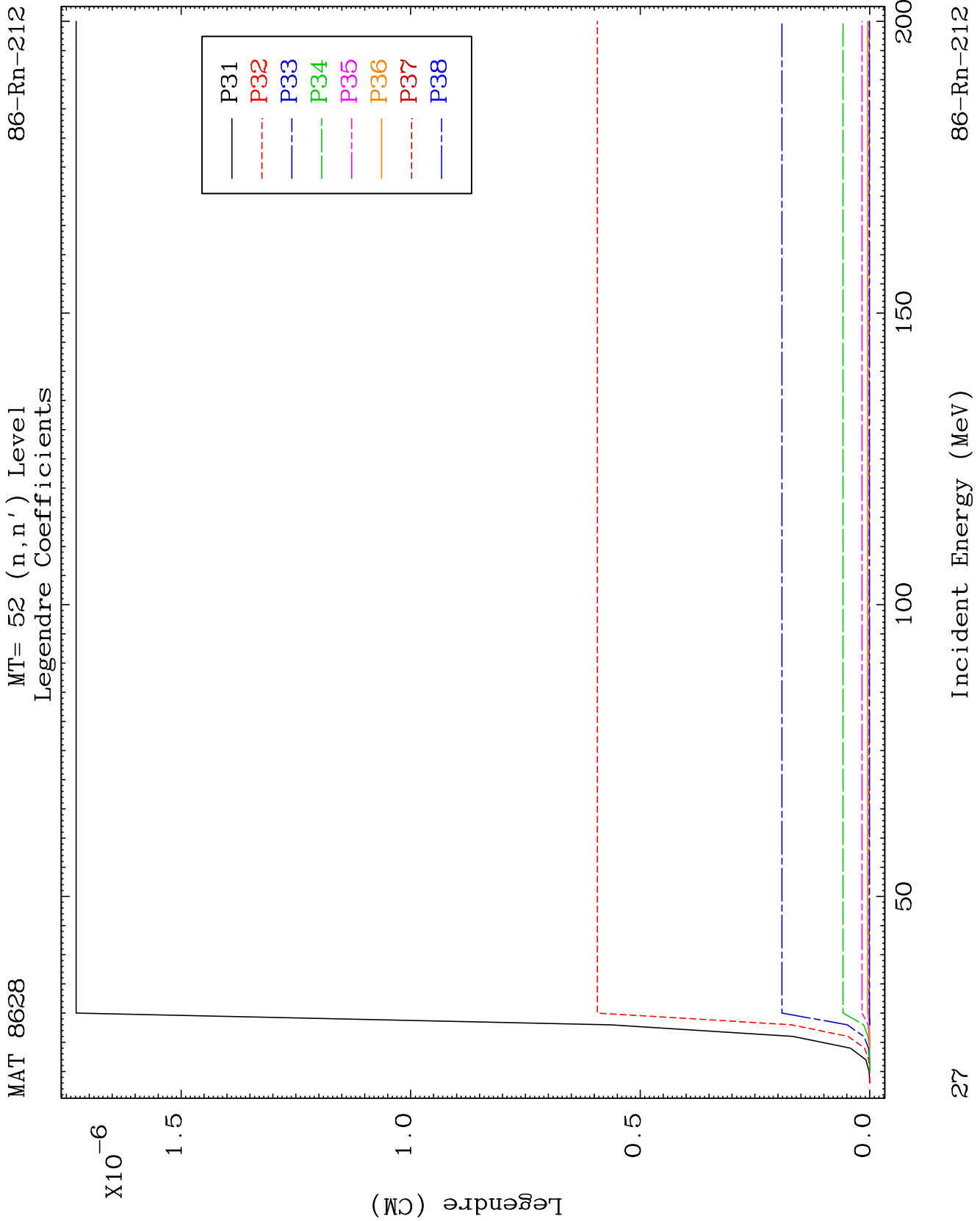
MAT 8628

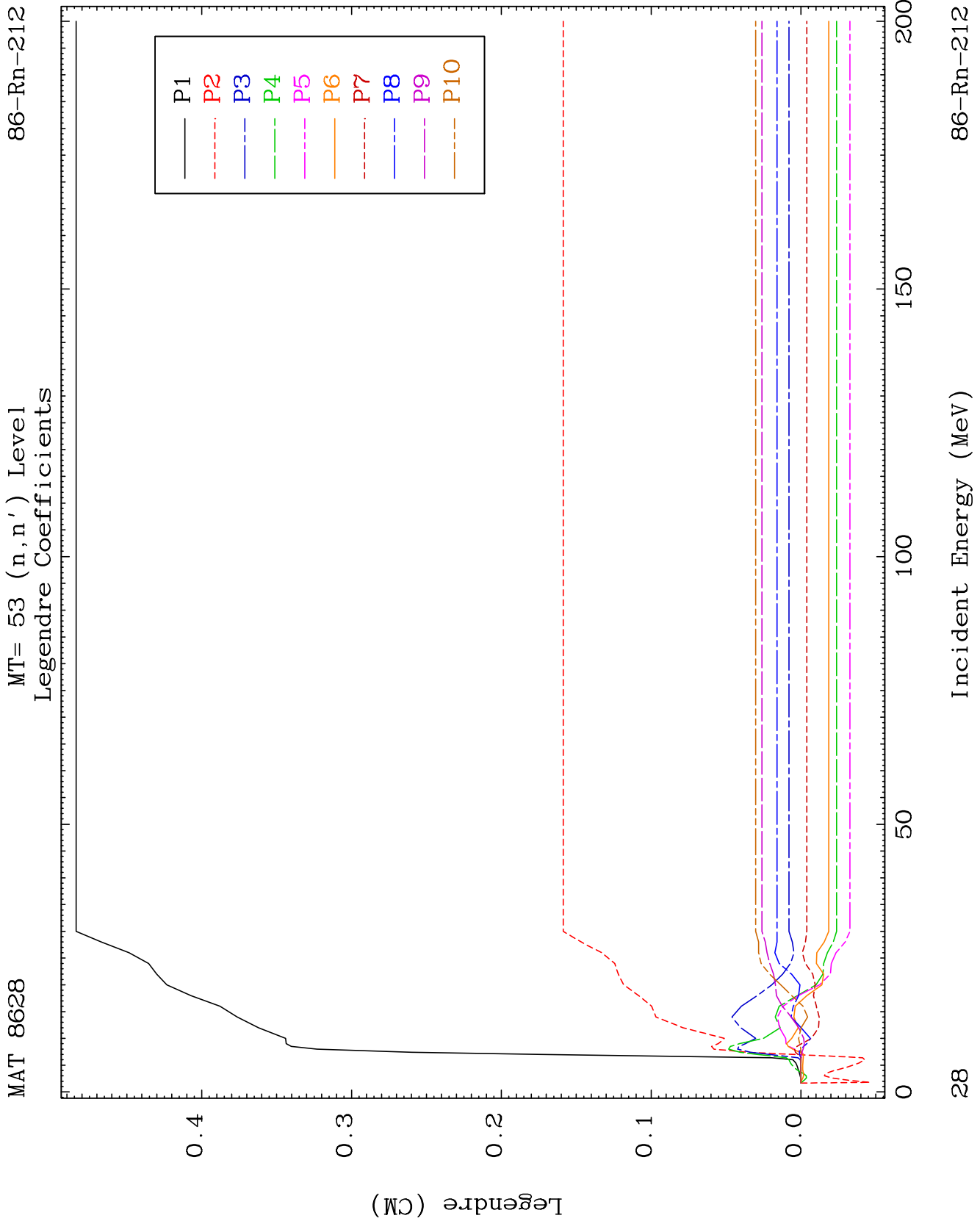
MT= 52 (n, n') Level
Legendre Coefficients

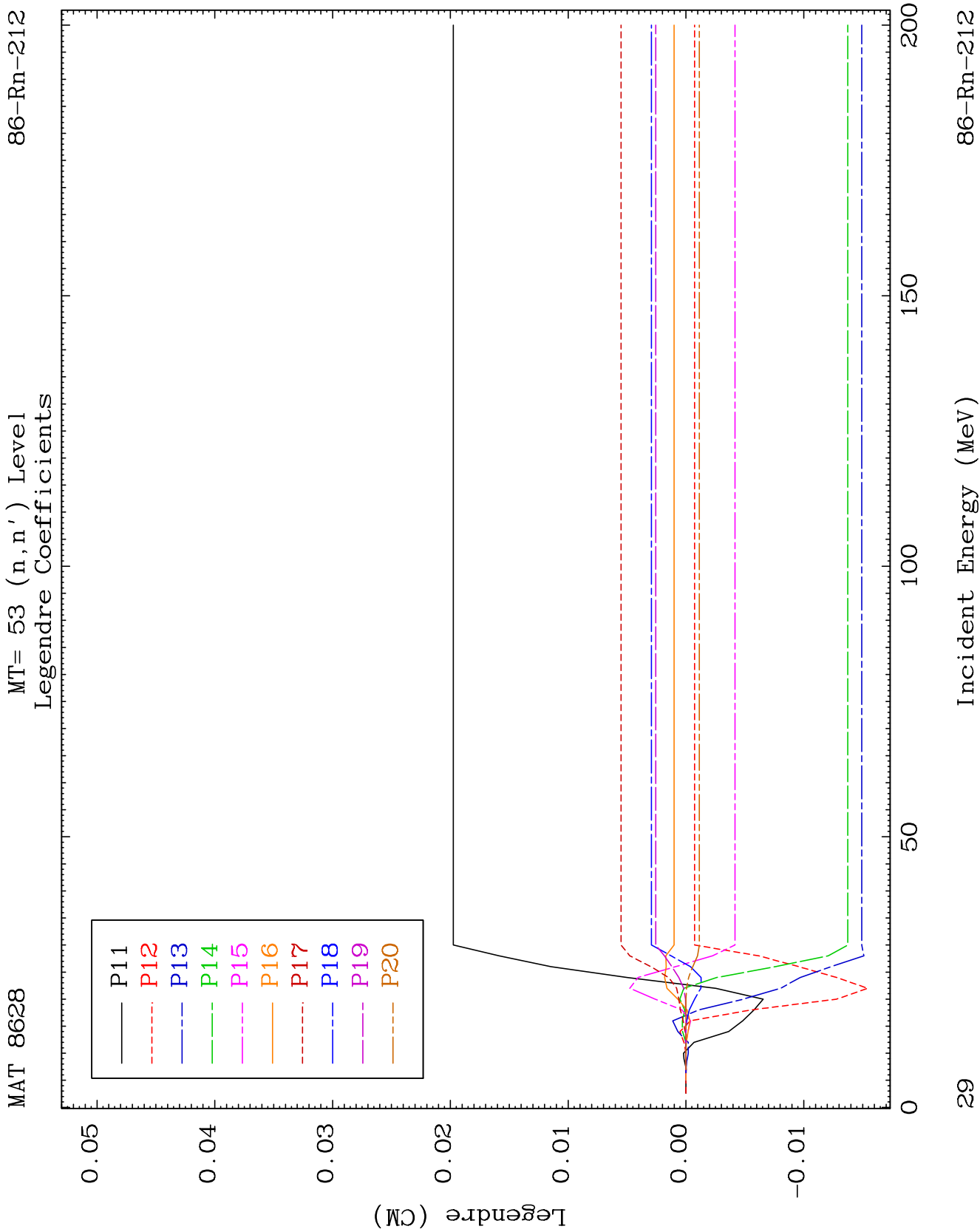
86-Rn-212

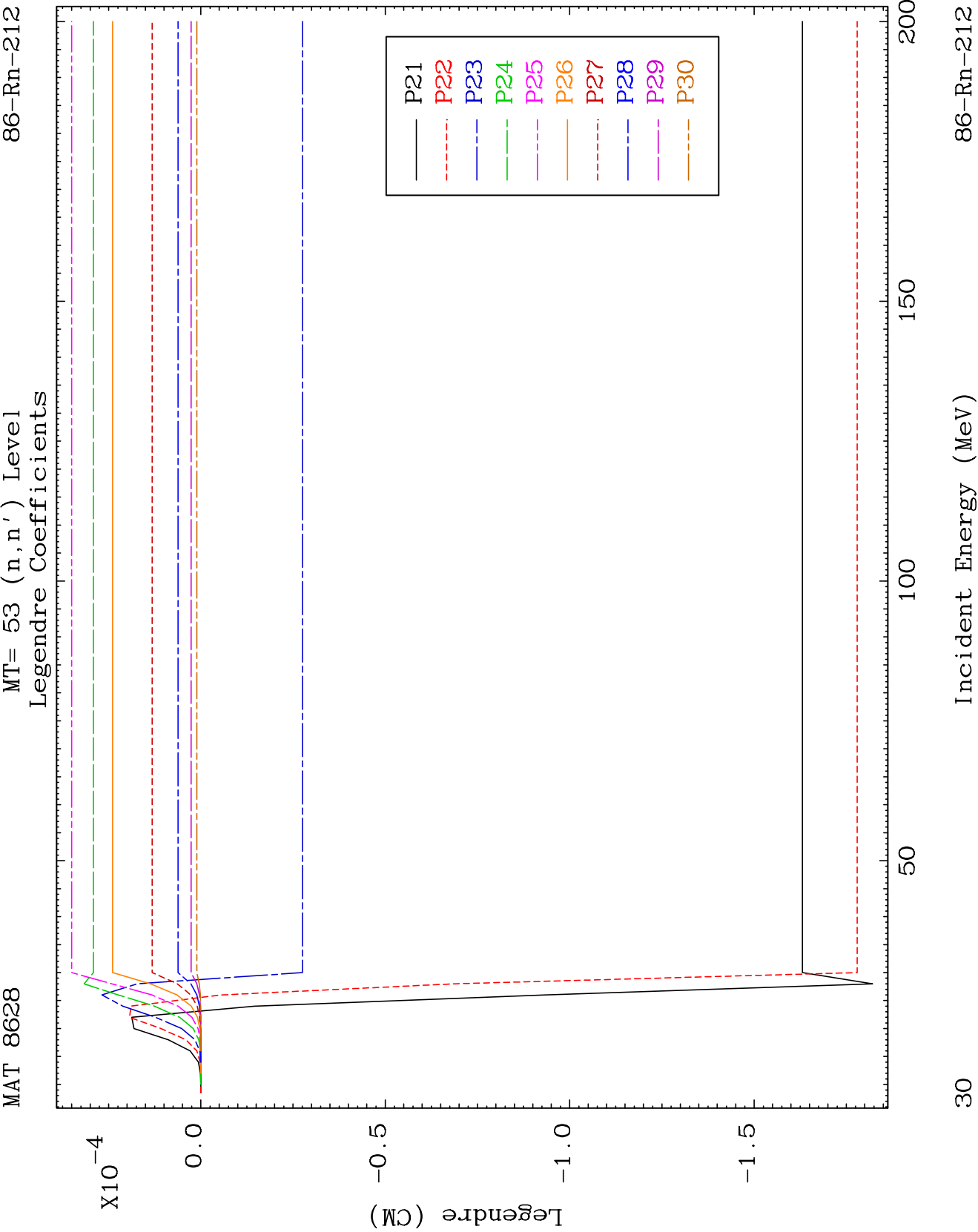


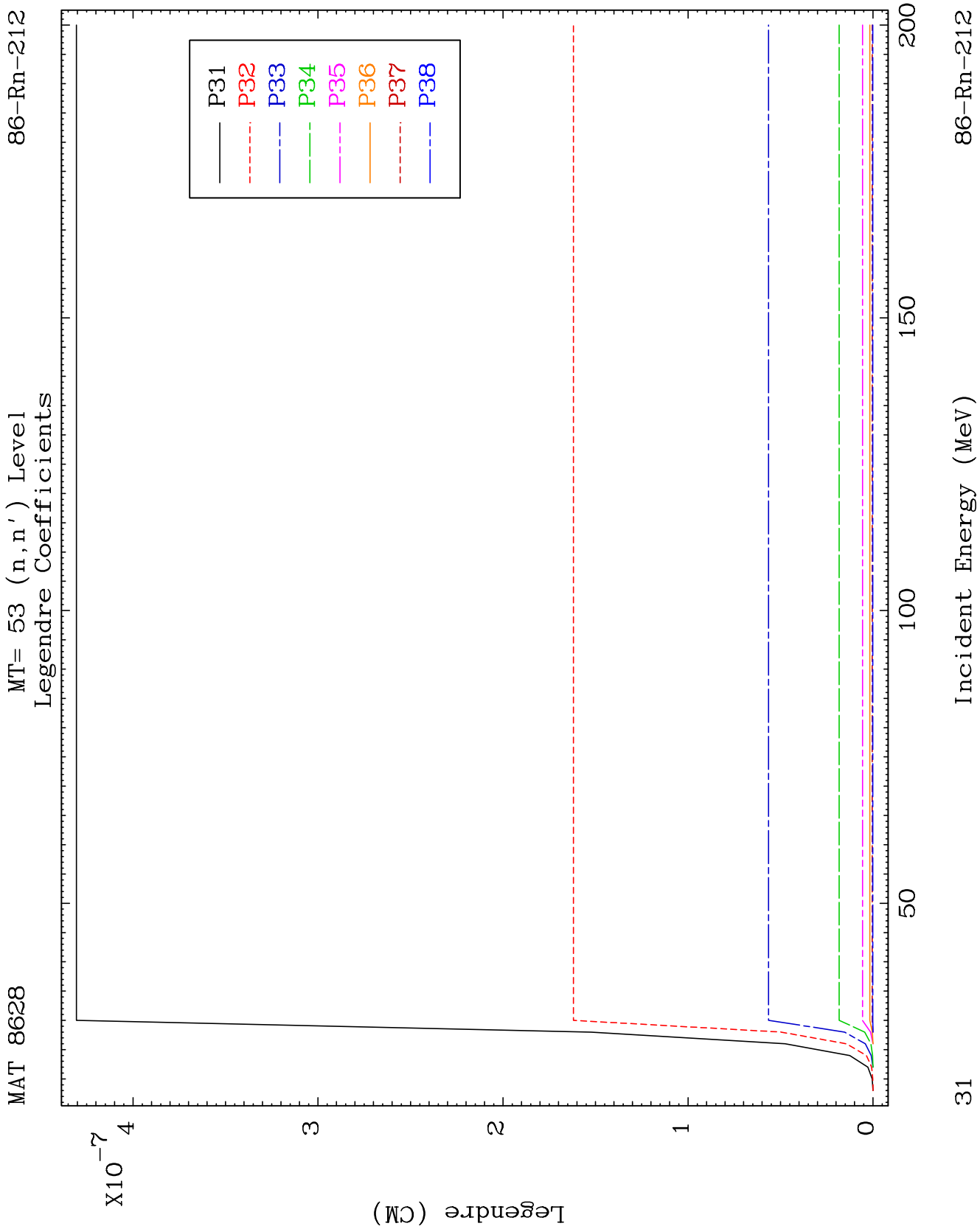


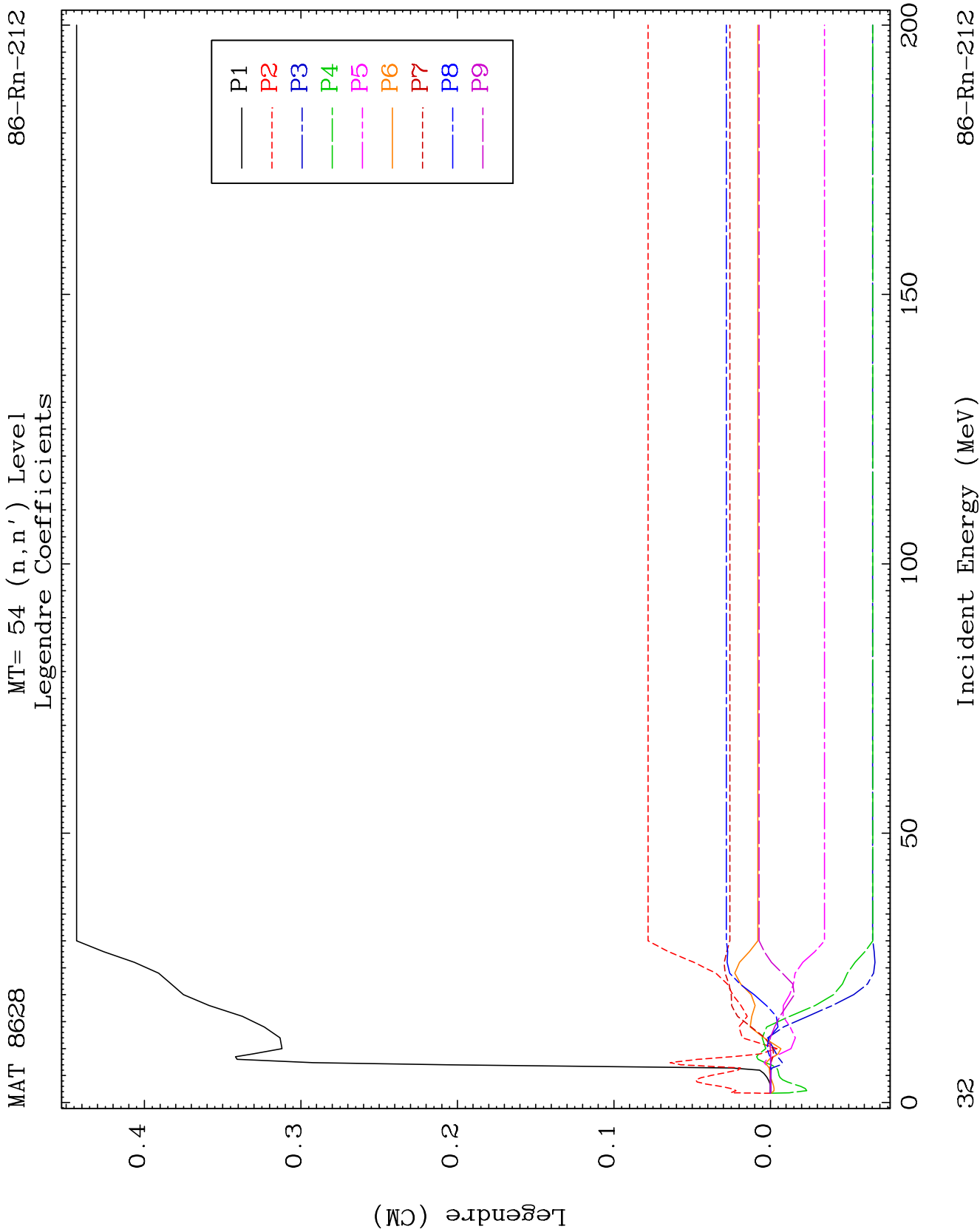


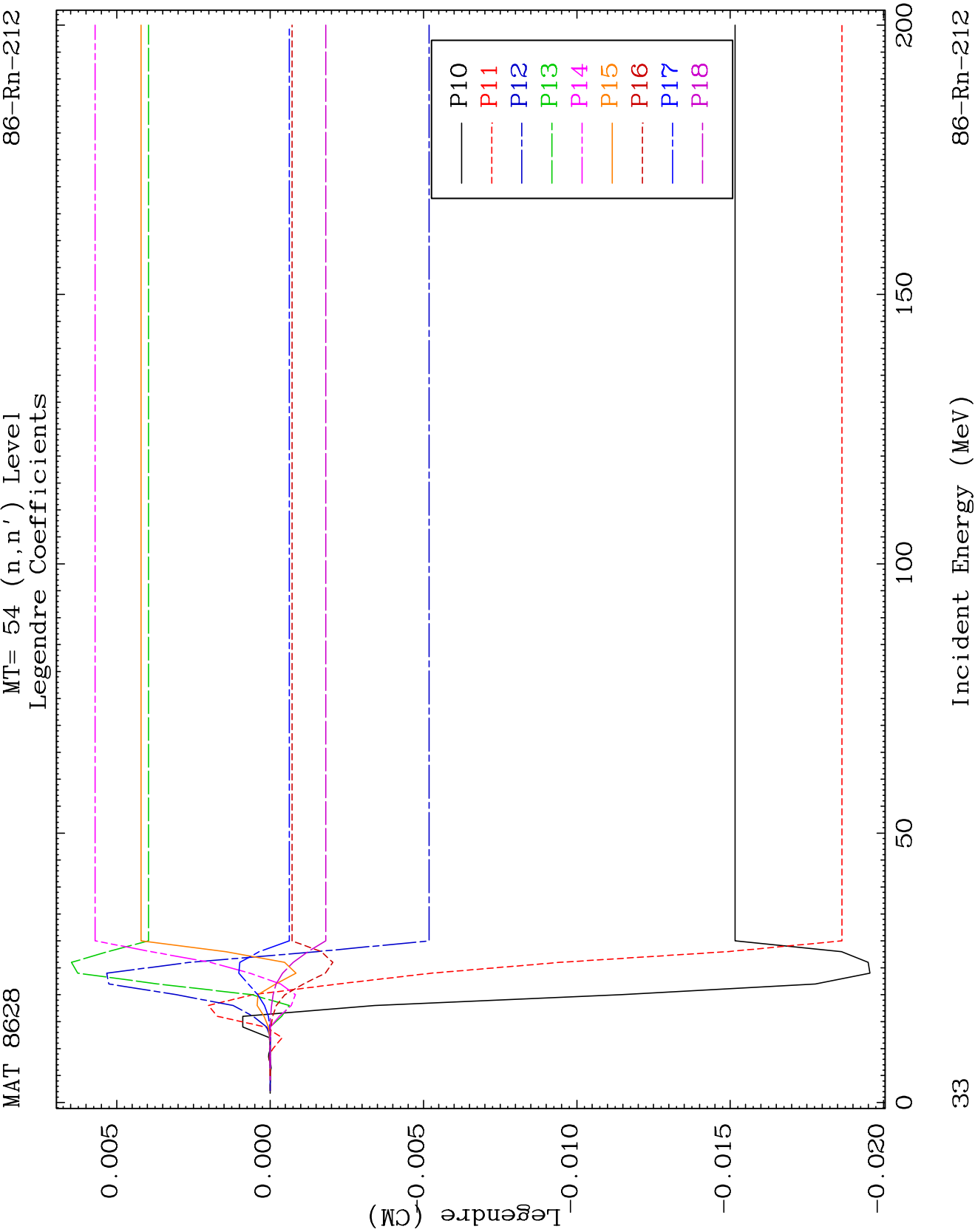


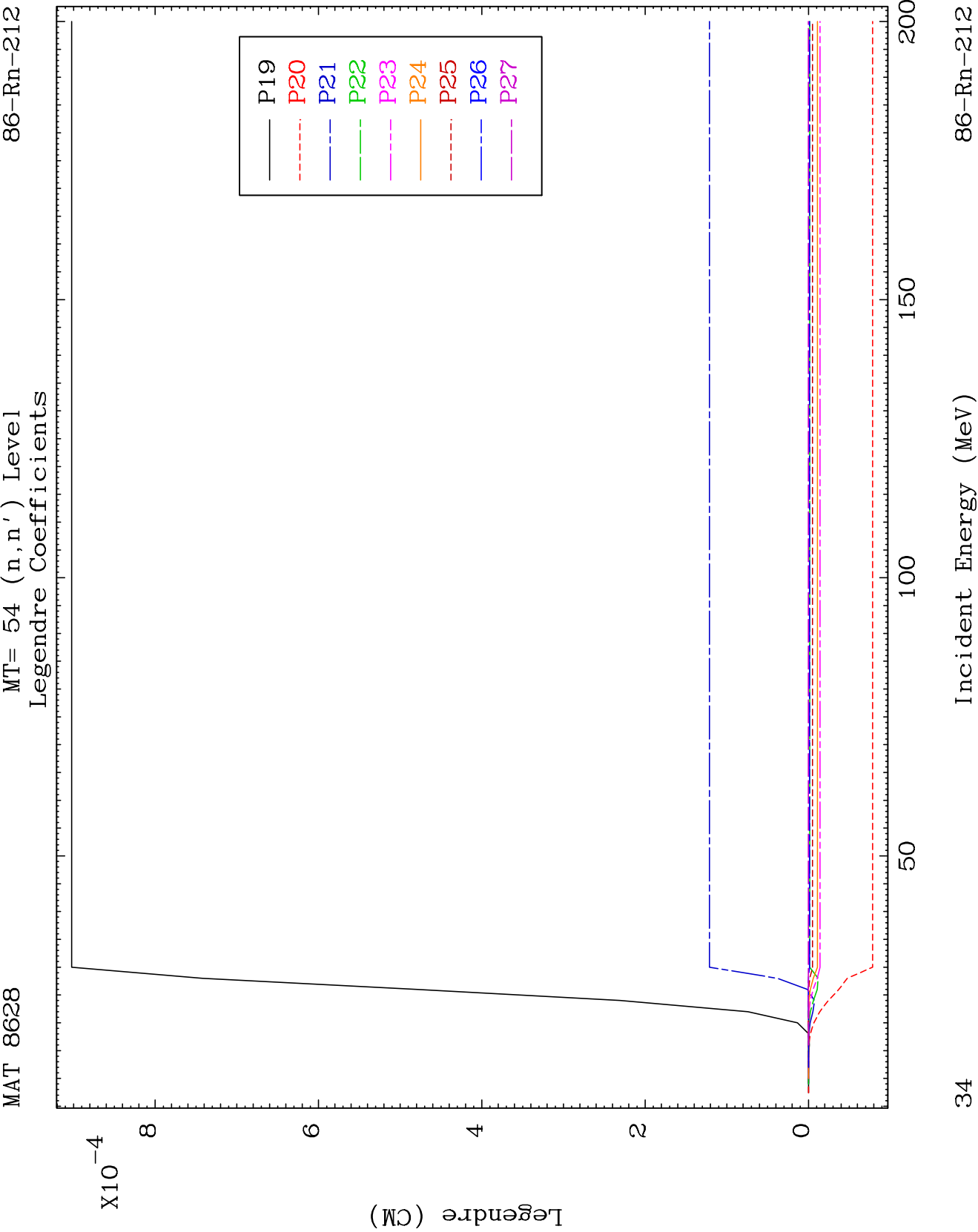


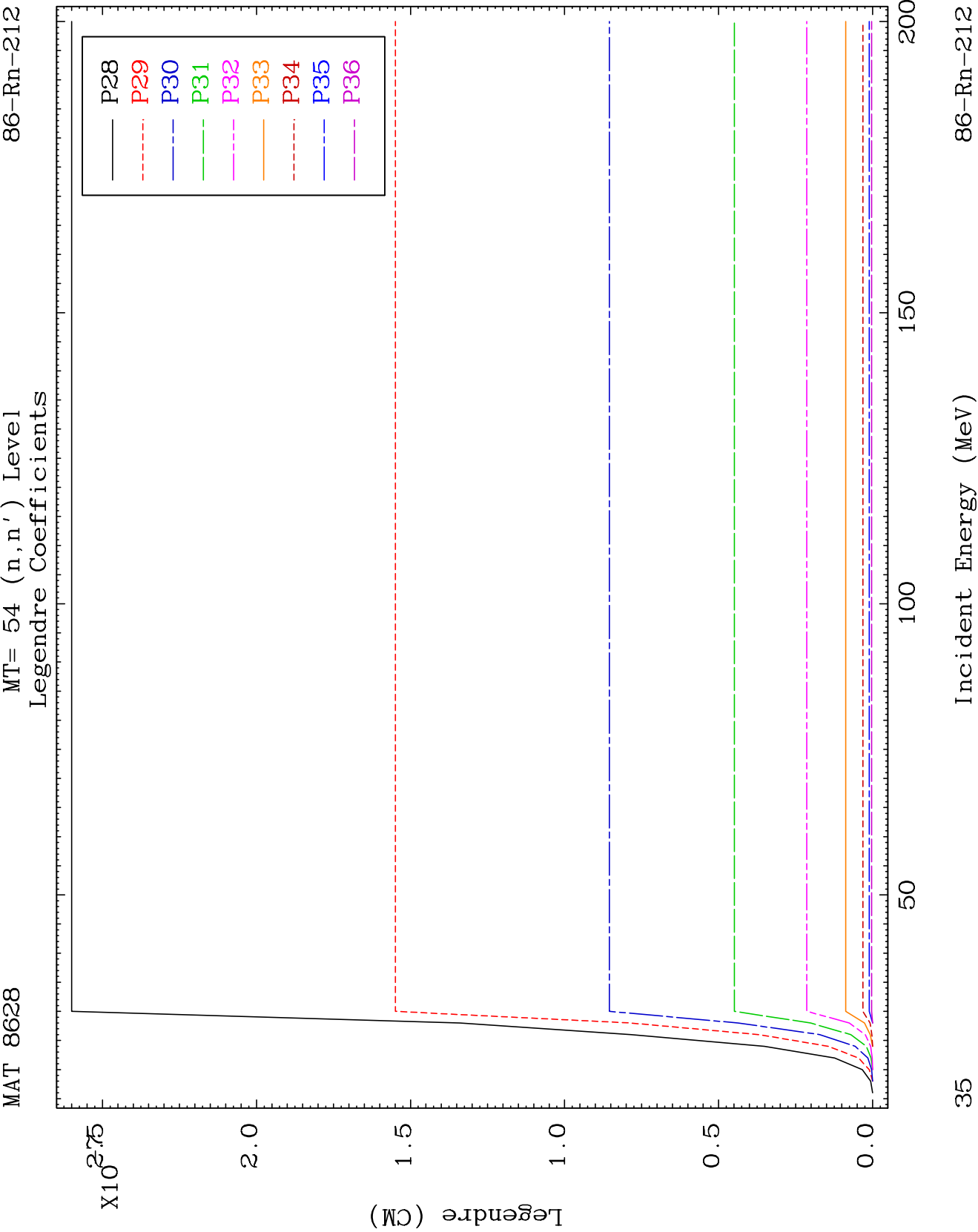








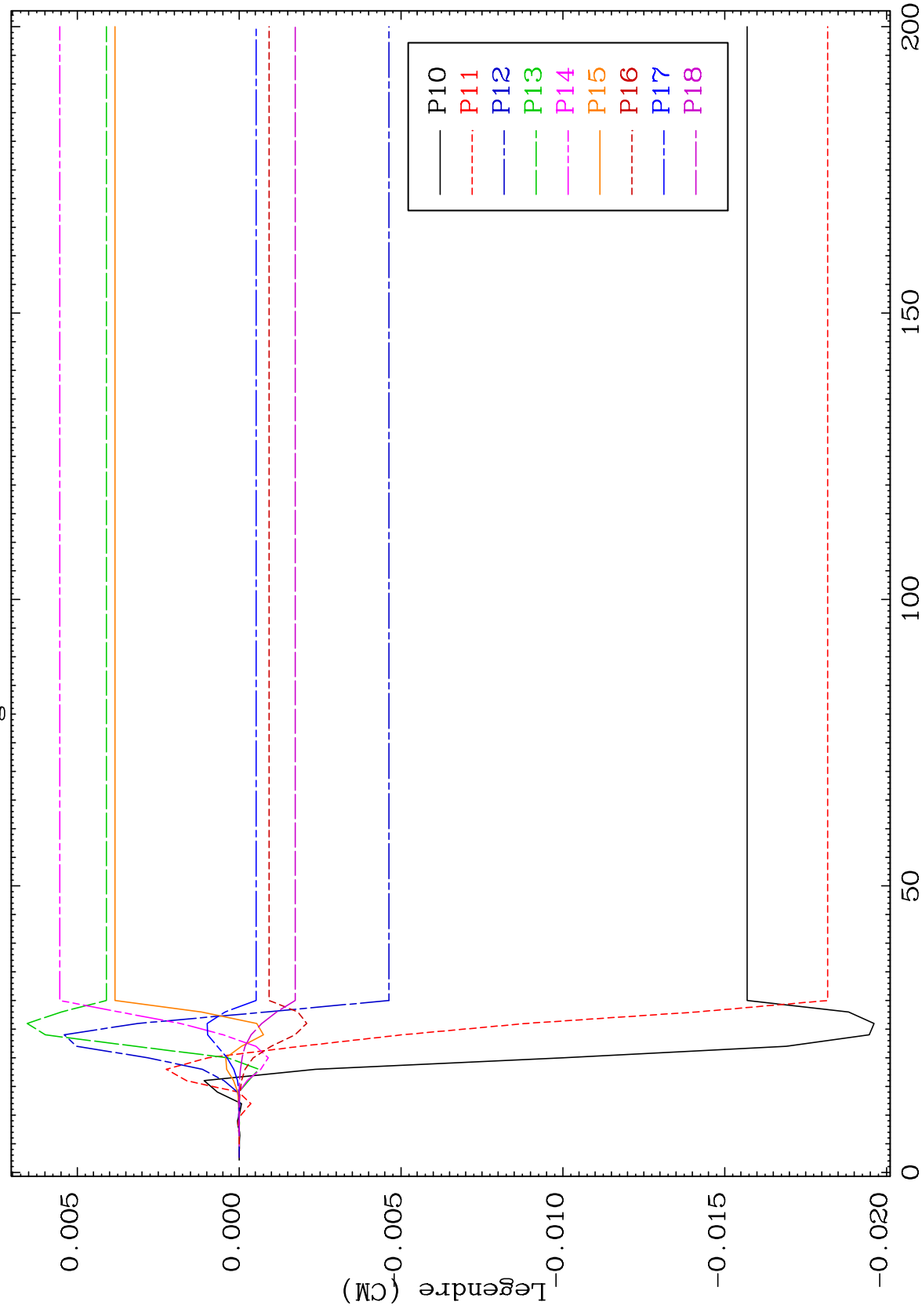




MAT 8628

86-Rn-212

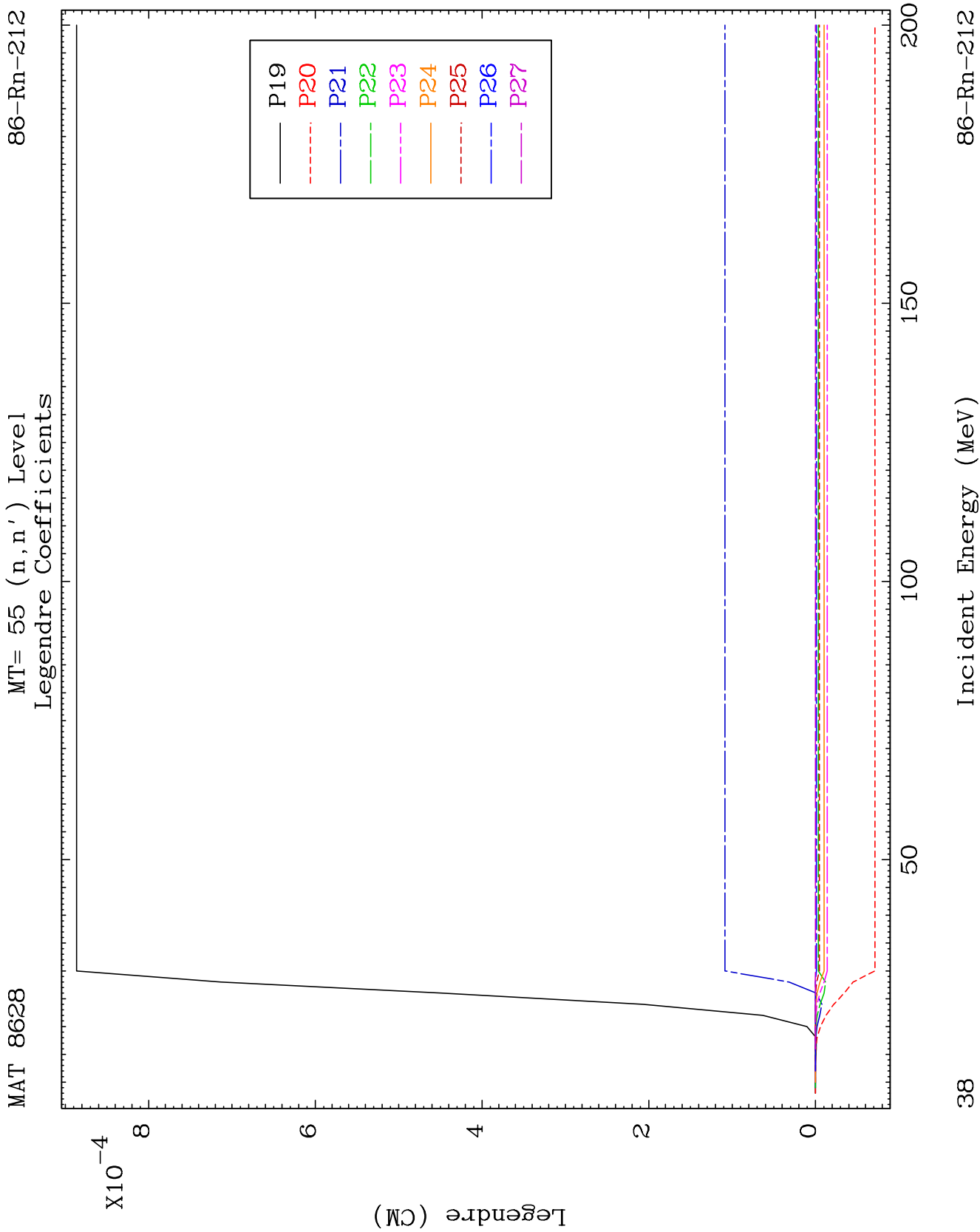
MT= 55 (n, n') Level
Legendre Coefficients



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Incident Energy (MeV)

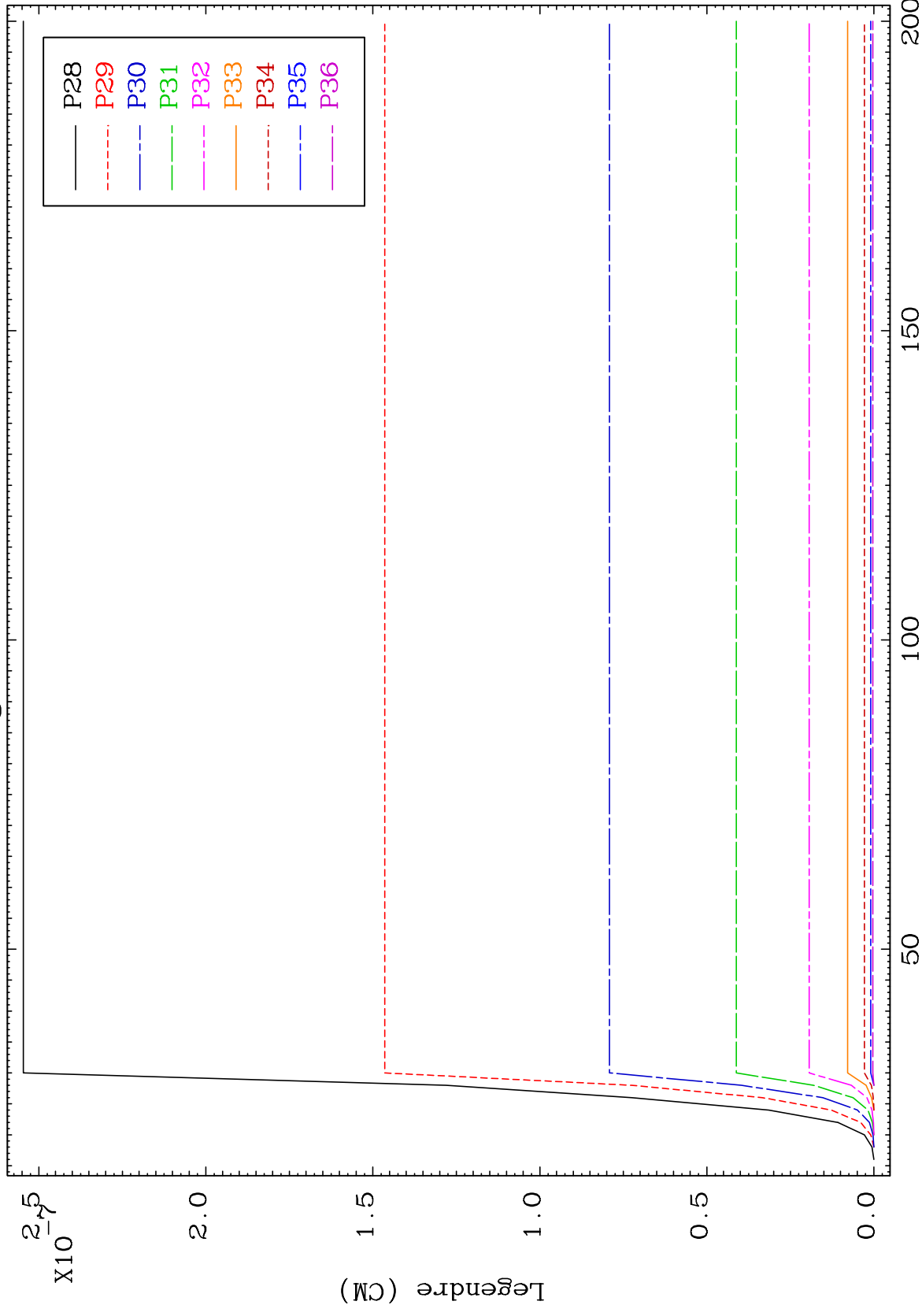
86-Rn-212



MAT 8628

MT= 55 (n, n') Level
Legendre Coefficients

86-Rn-212



39

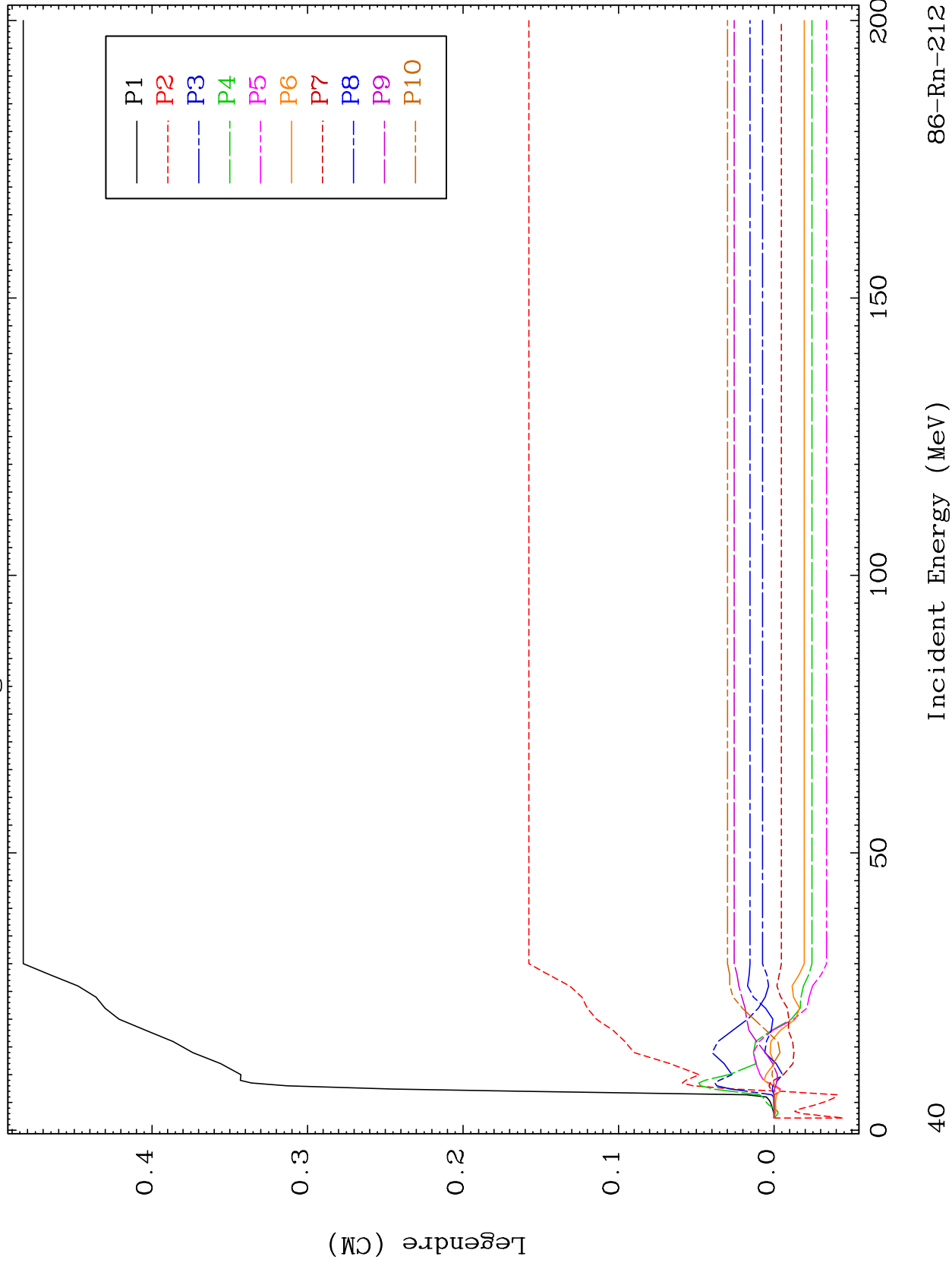
Incident Energy (MeV)

86-Rn-212

MAT 8628

MT= 56 (n, n') Level
Legendre Coefficients

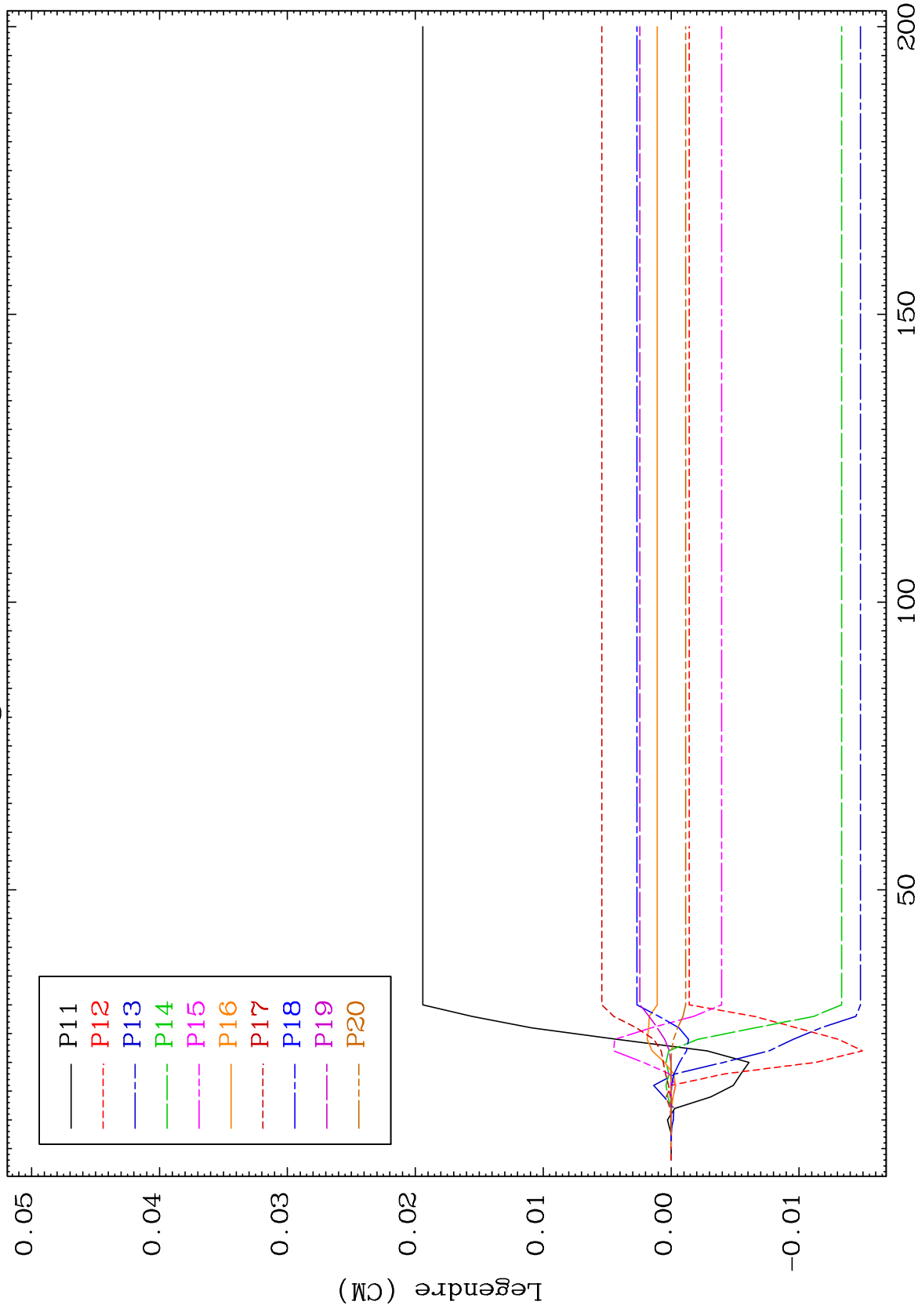
86-Rn-212



MAT 8628

MT= 56 (n, n') Level
Legendre Coefficients

86-Rn-212



41

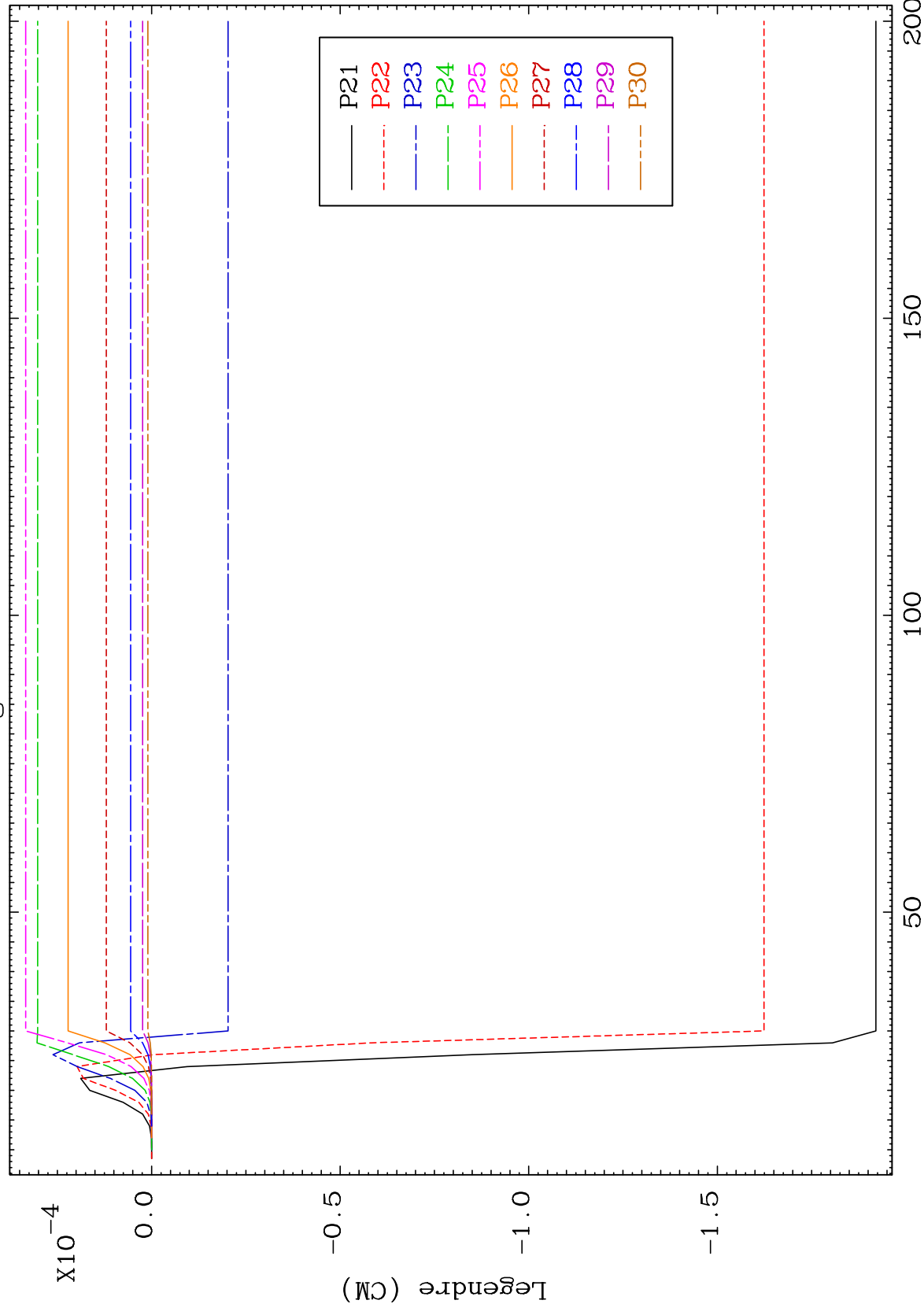
Incident Energy (MeV)

86-Rn-212

MAT 8628

MT= 56 (n, n') Level
Legendre Coefficients

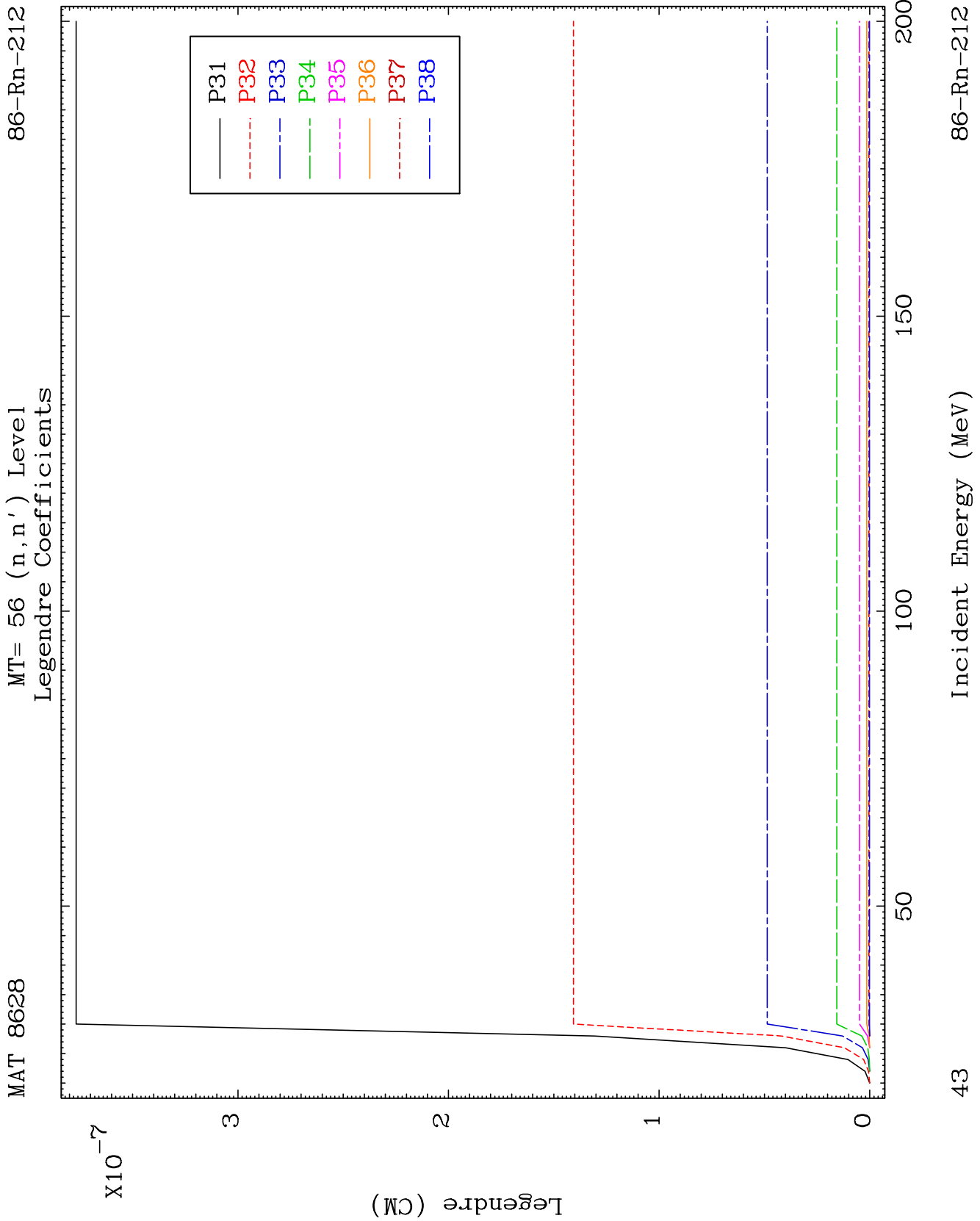
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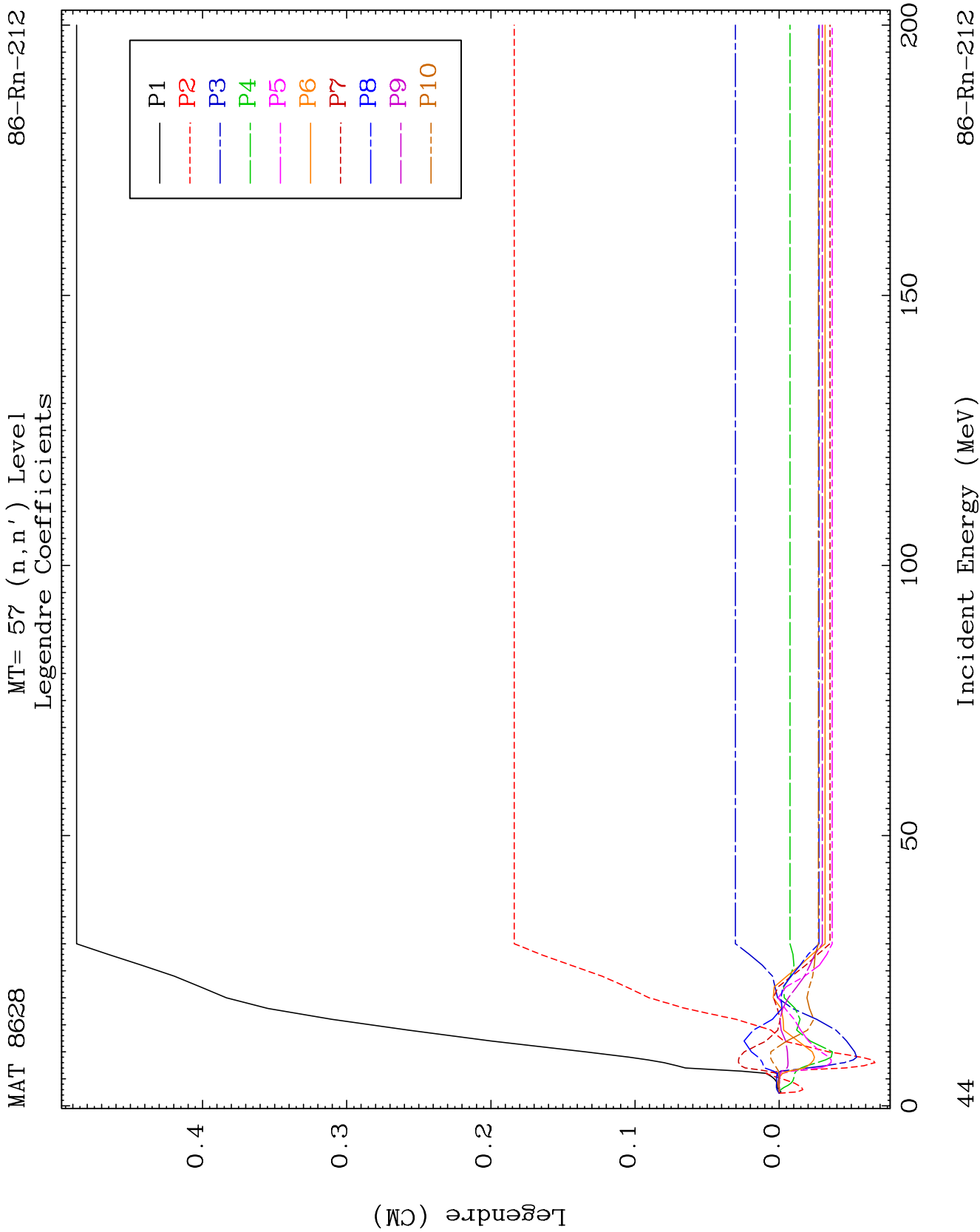


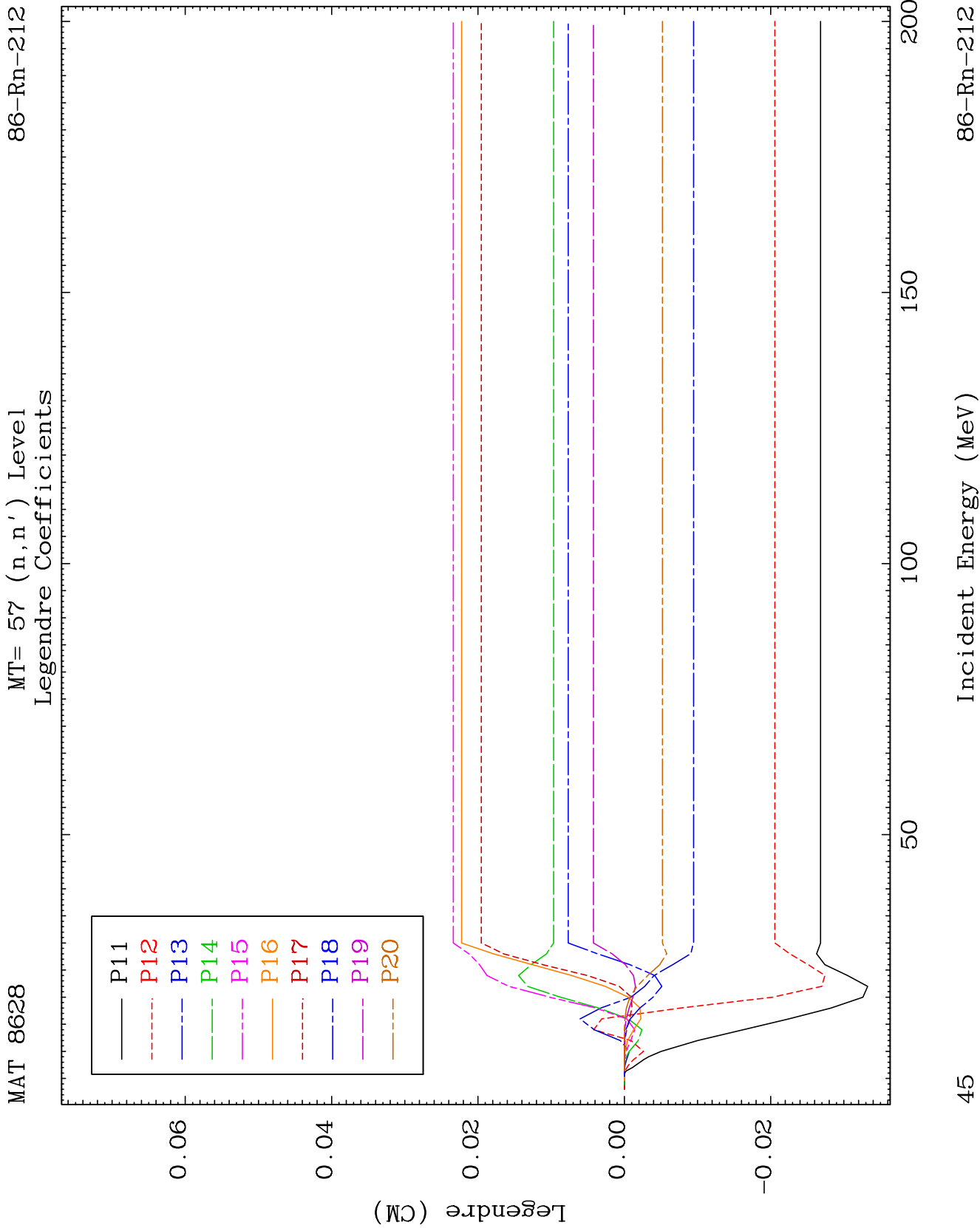
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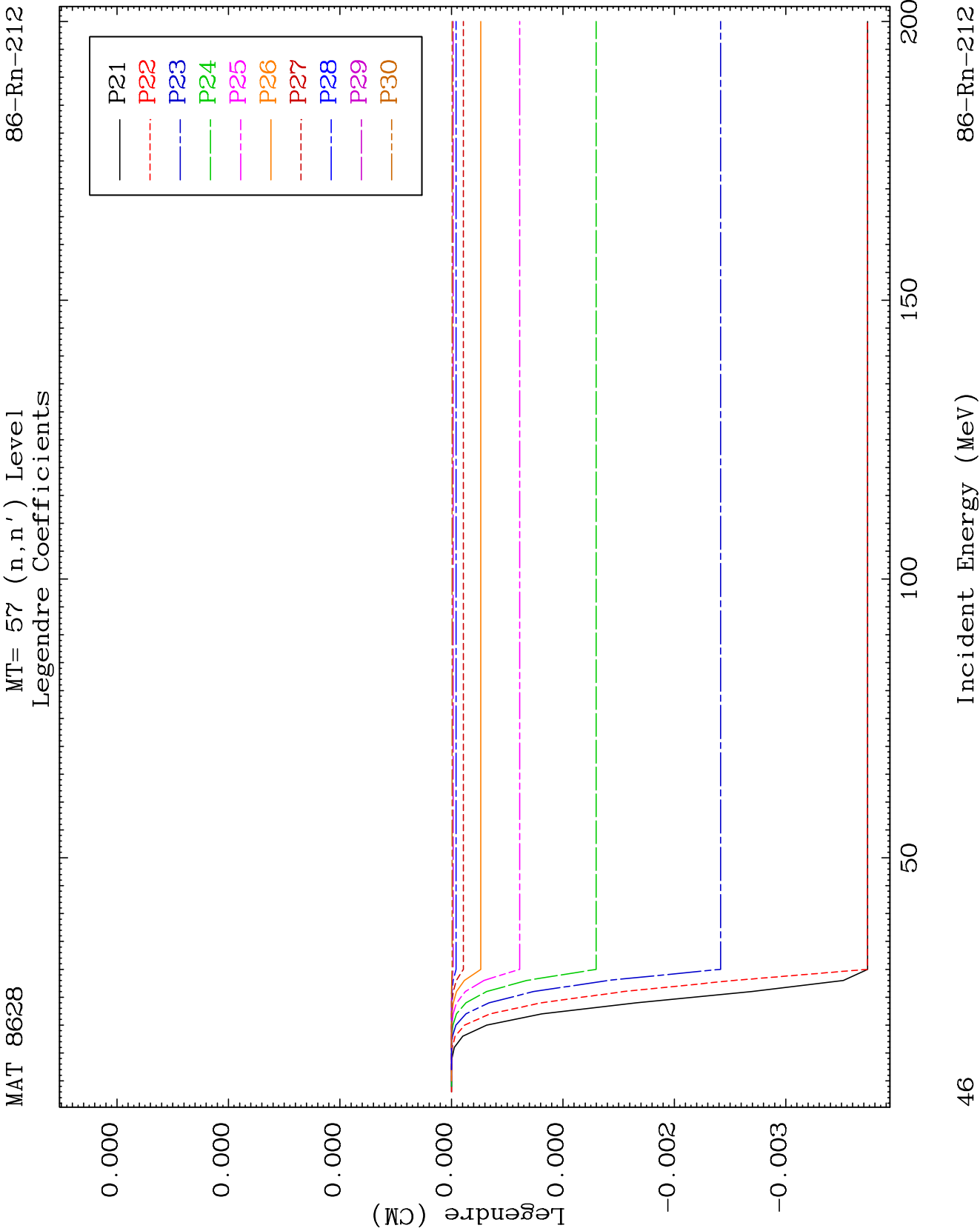
Incident Energy (MeV)

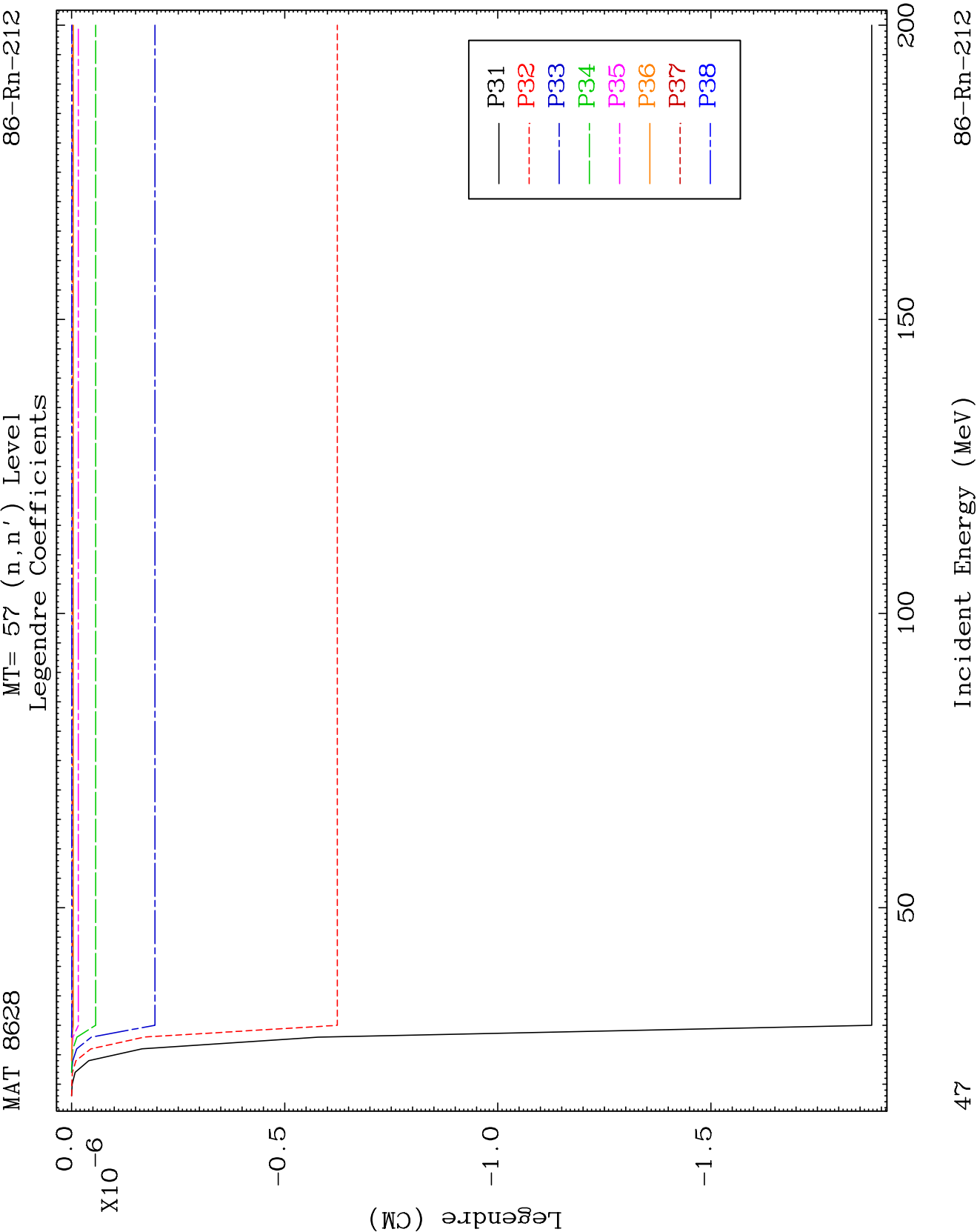
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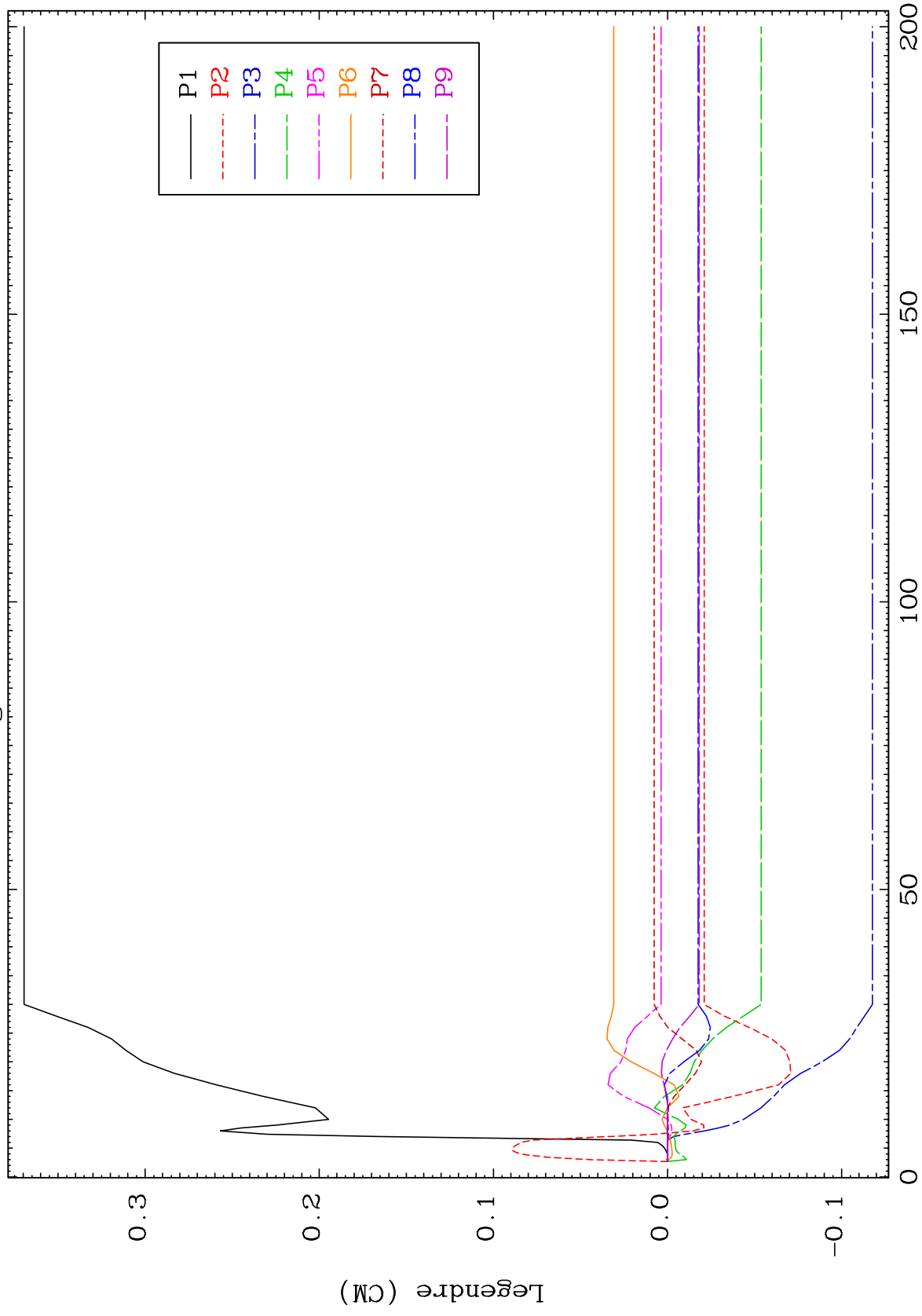




MAT 8628

MT= 58 (n,n') Level
Legendre Coefficients

86-Rn-212



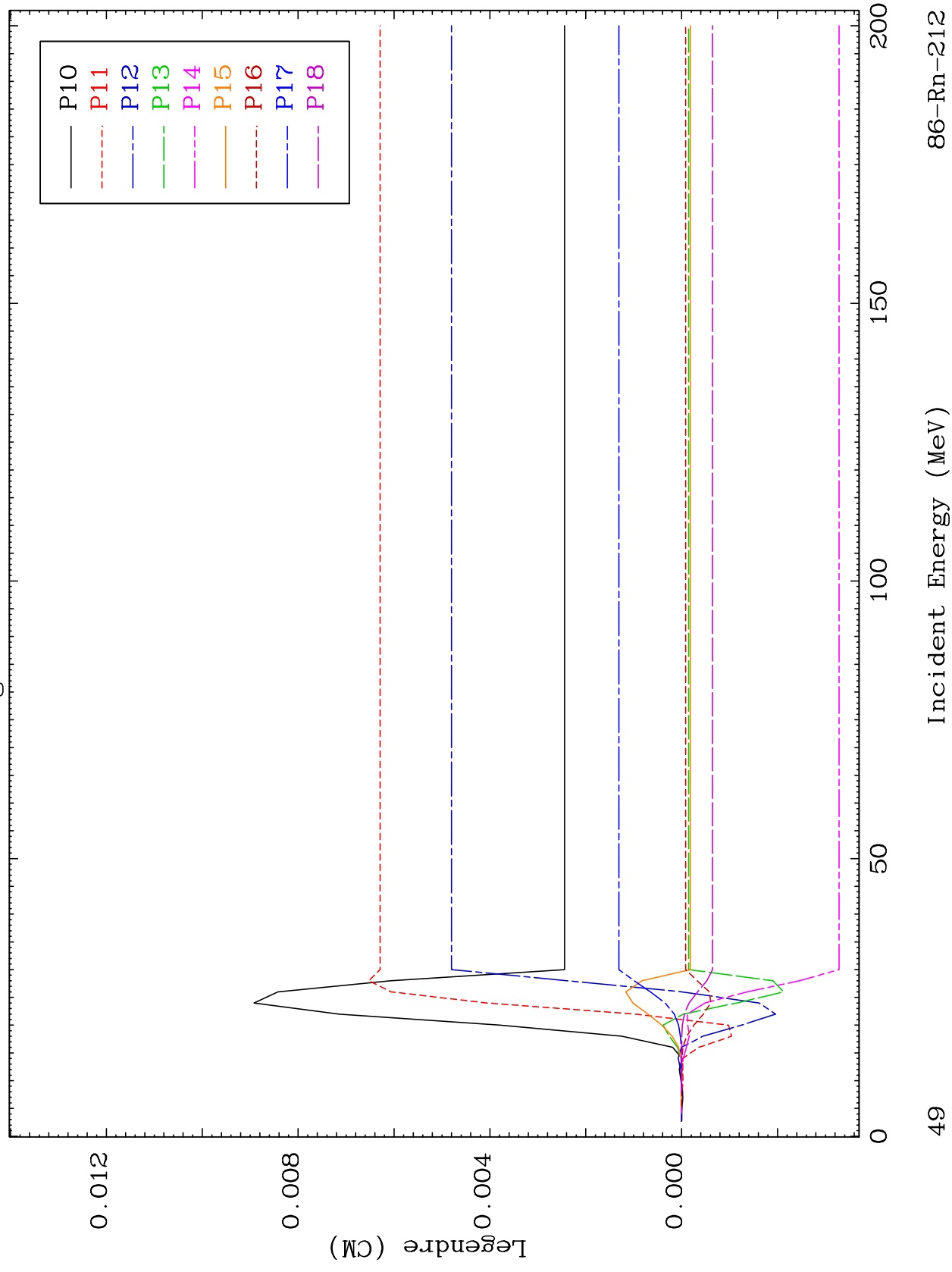
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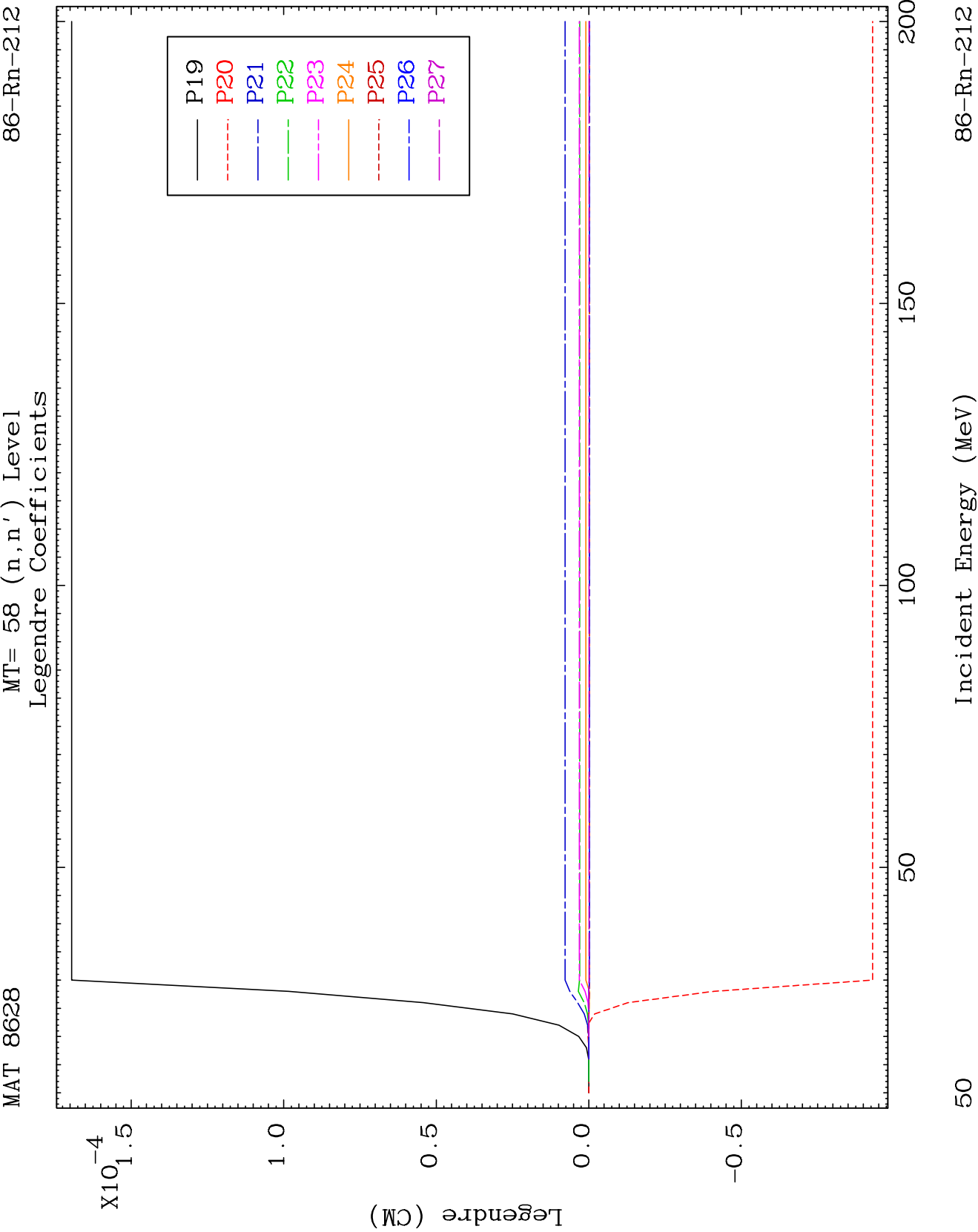
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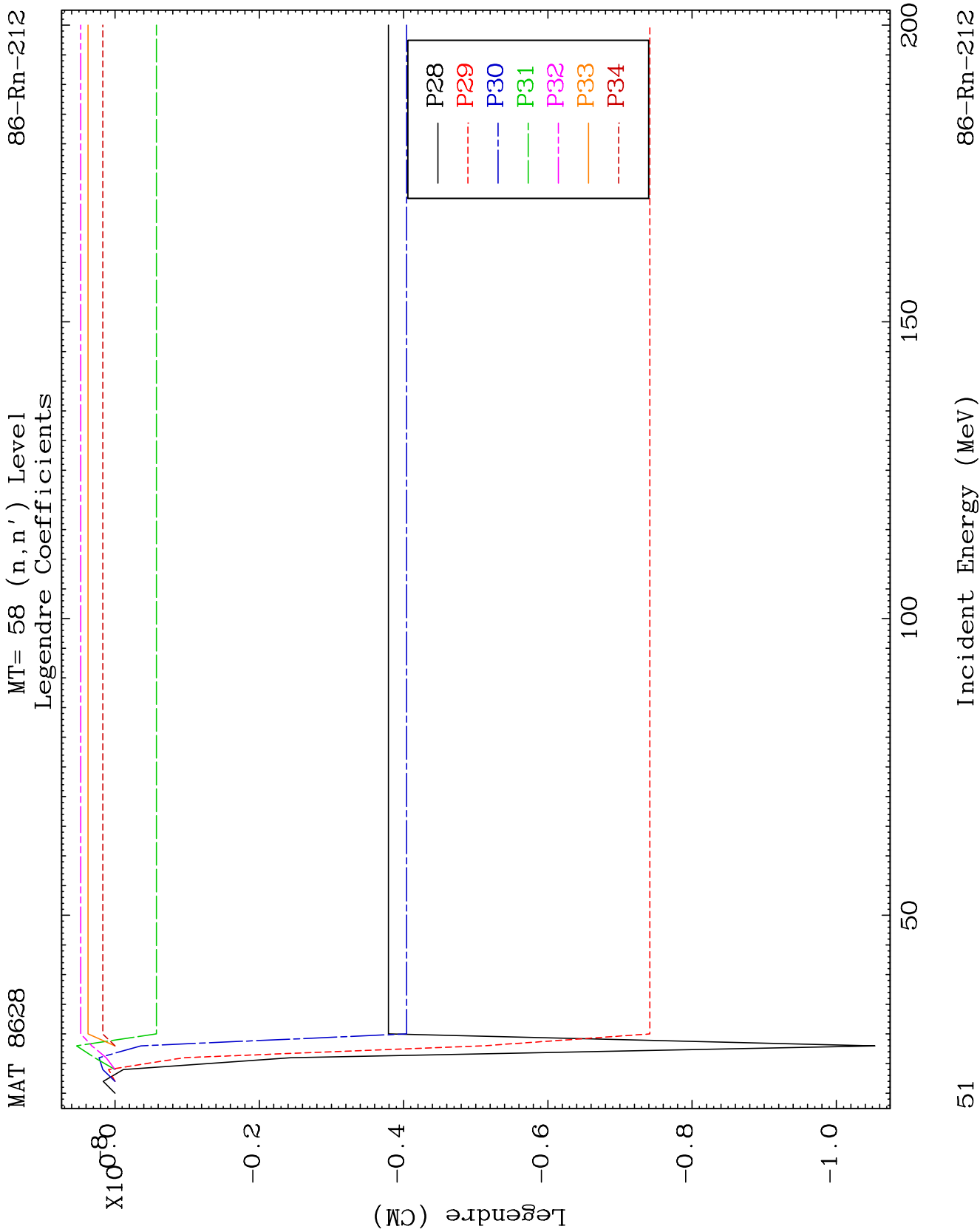
MAT 8628

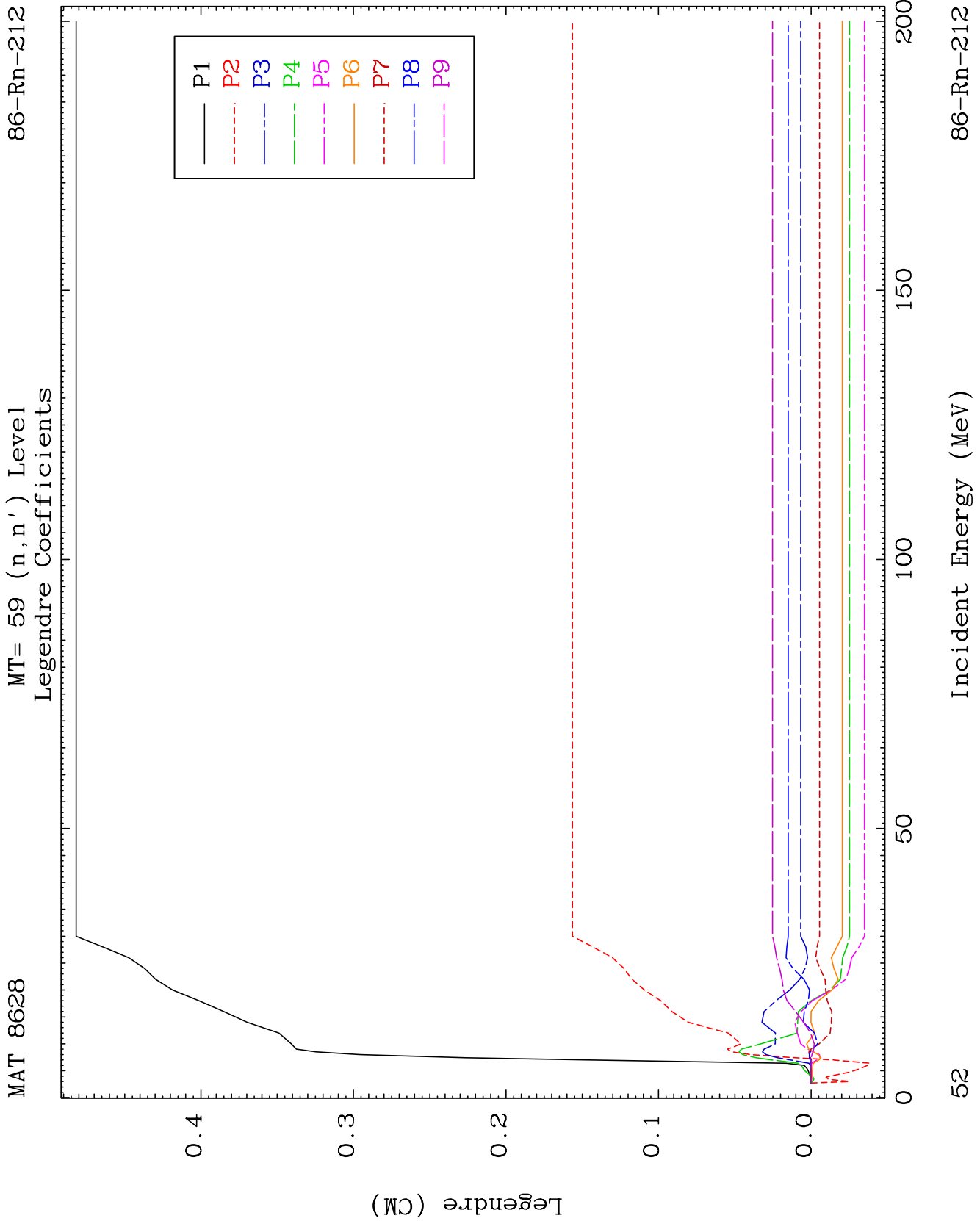
MT= 58 (n, n') Level
Legendre Coefficients

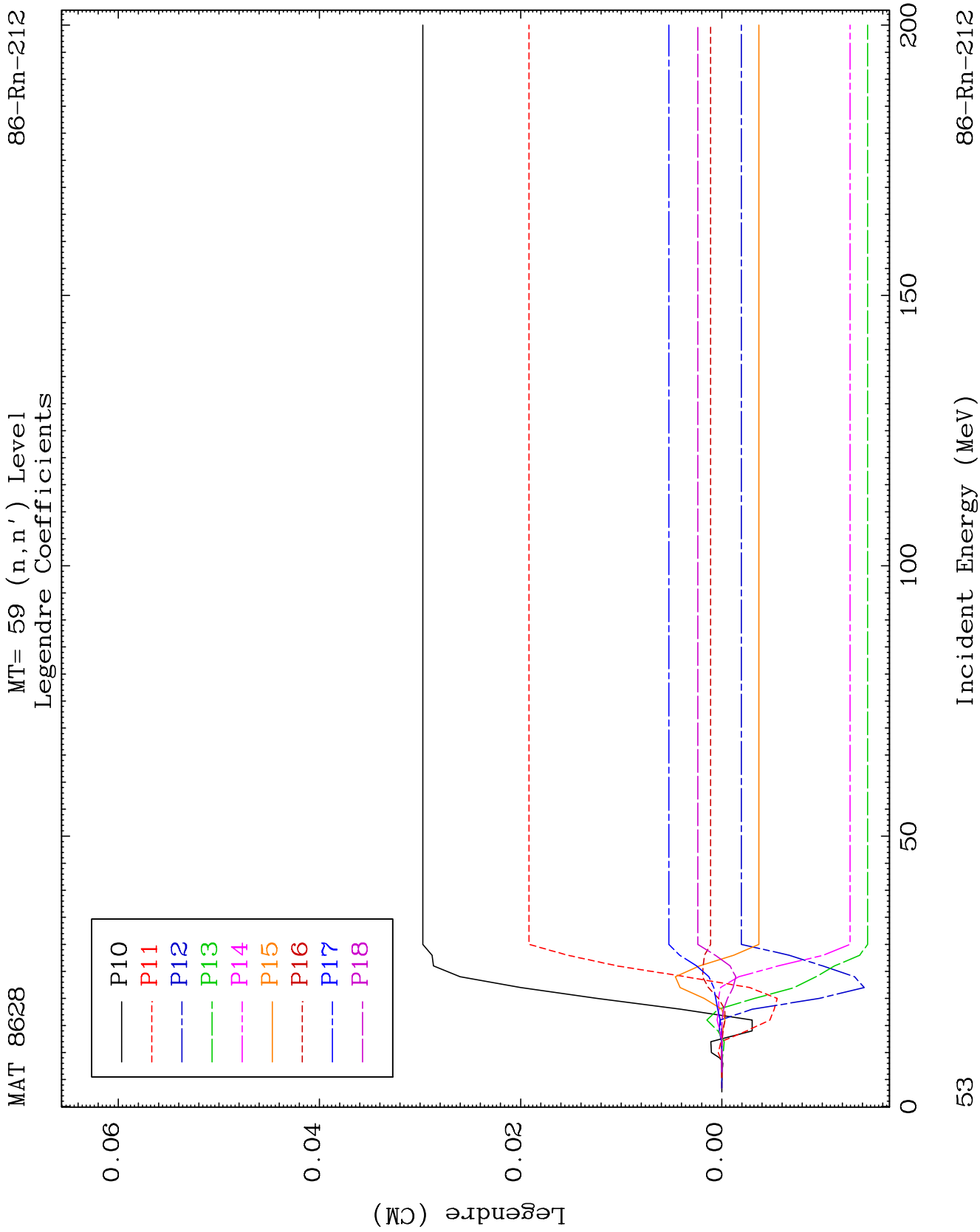
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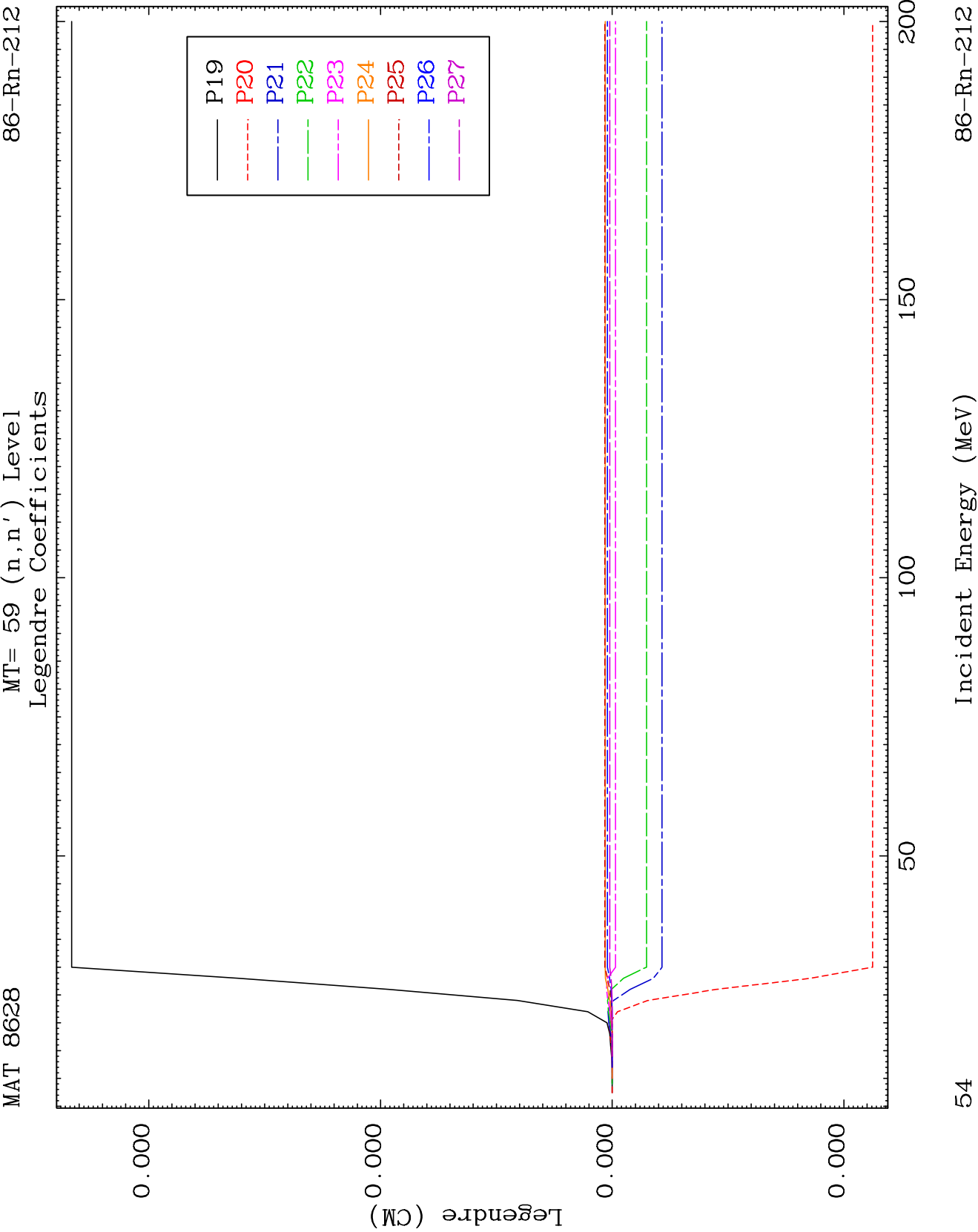


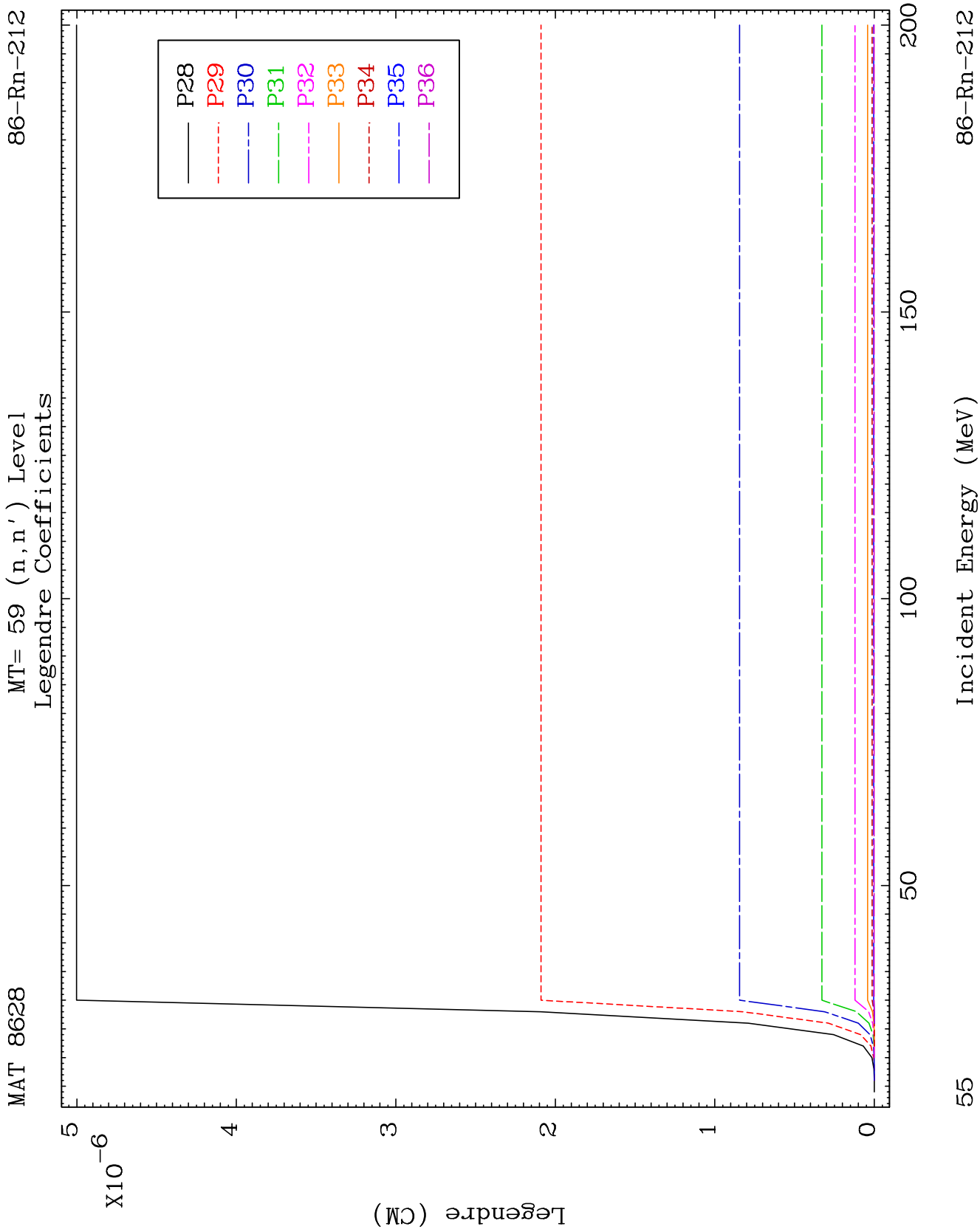


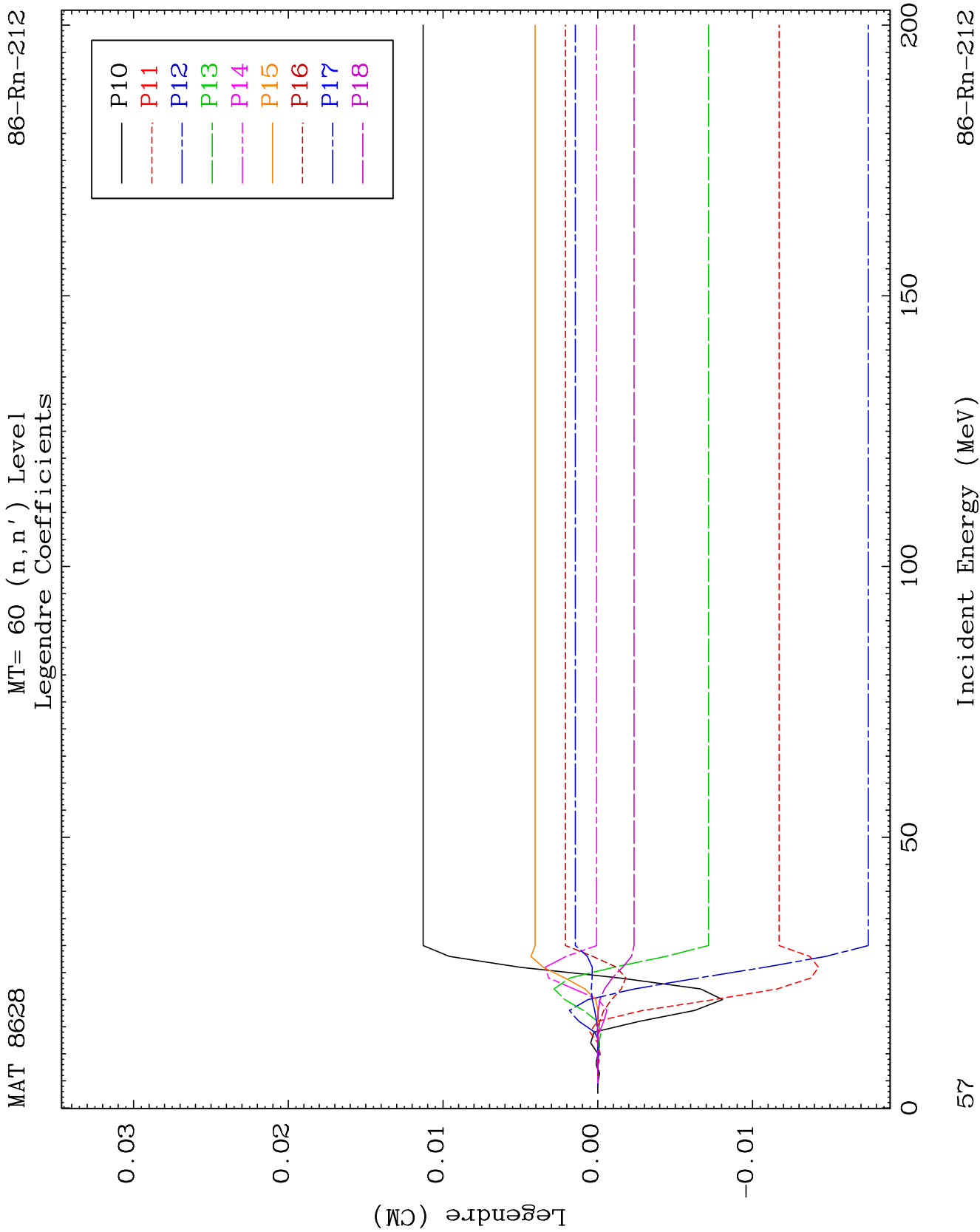


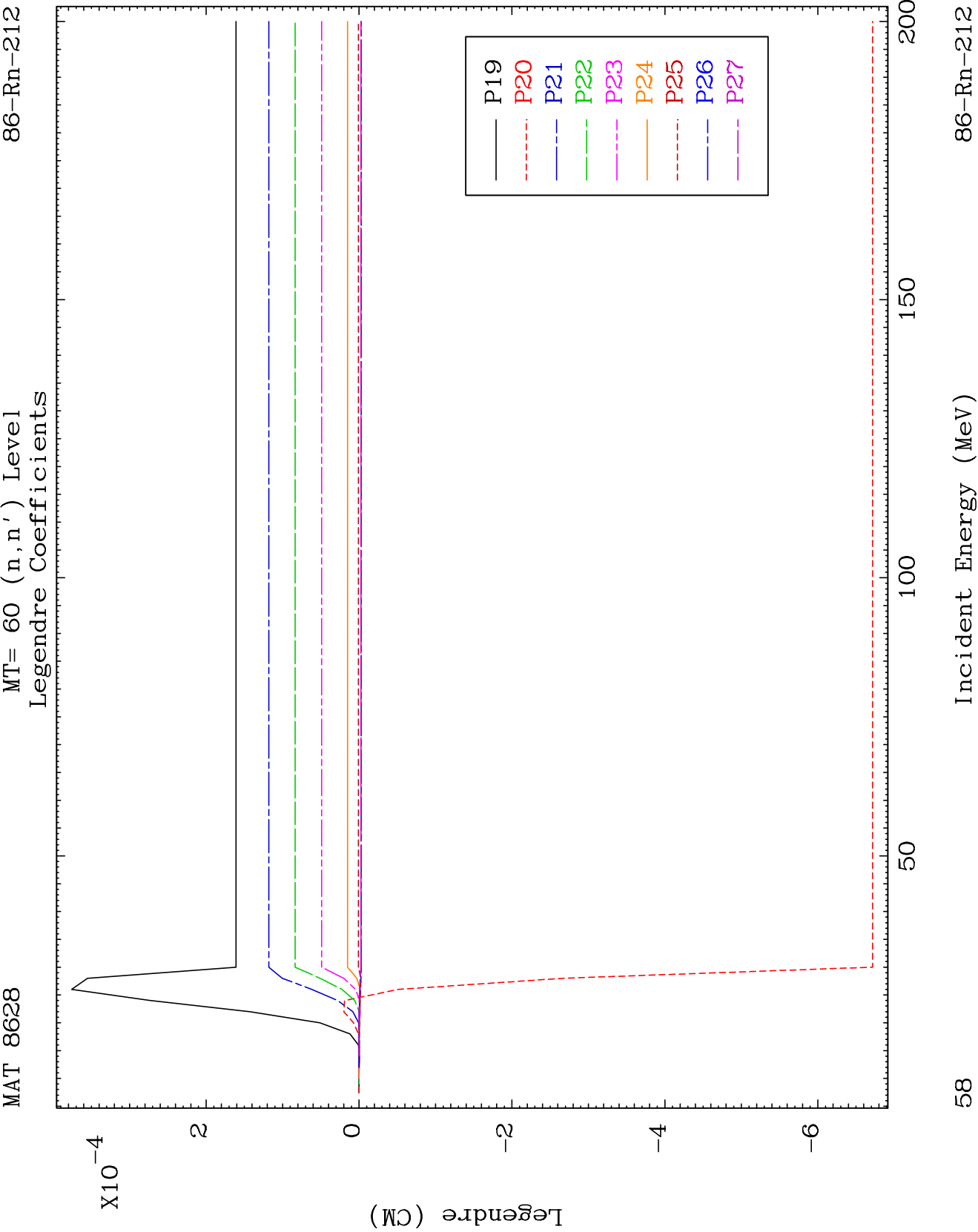


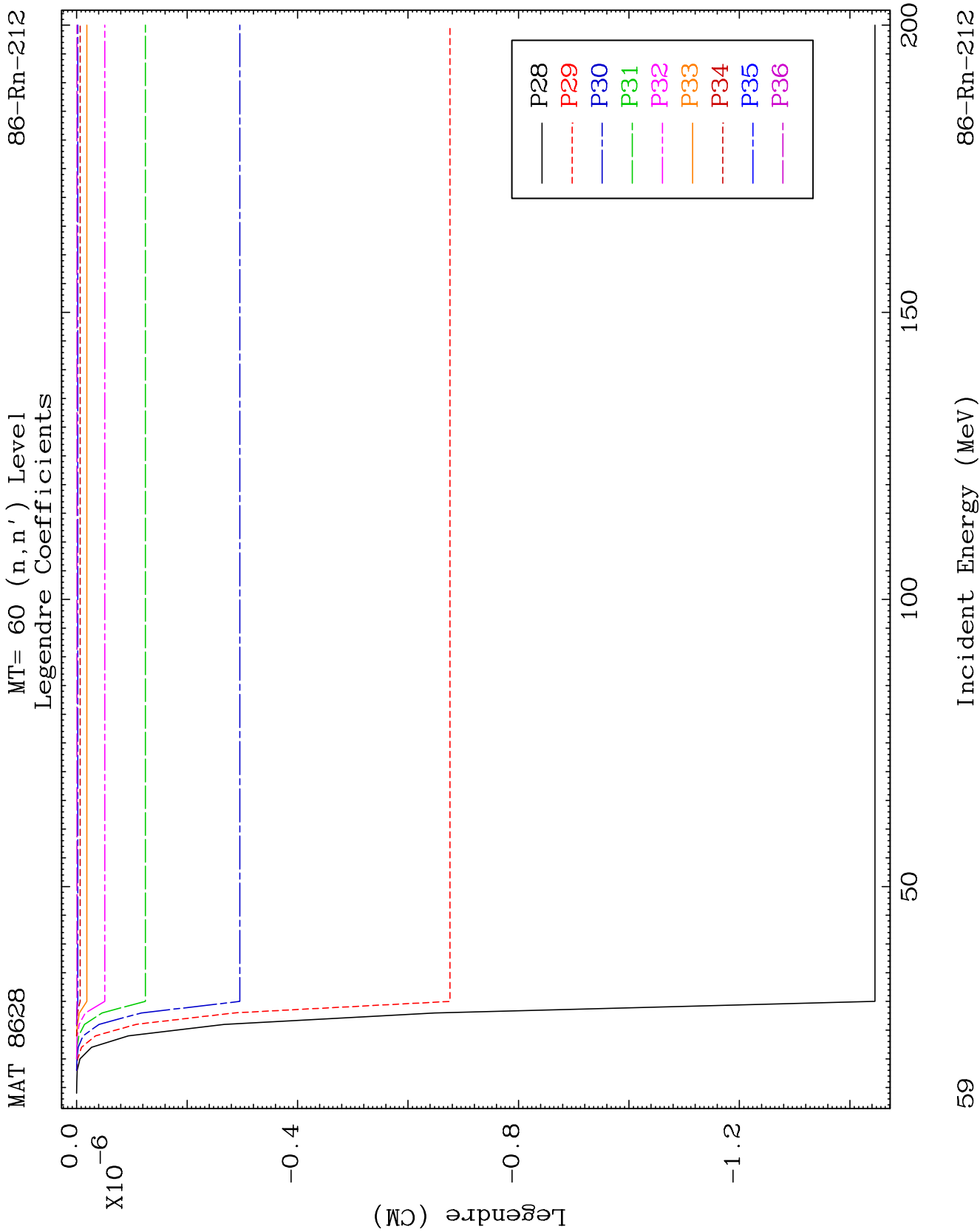








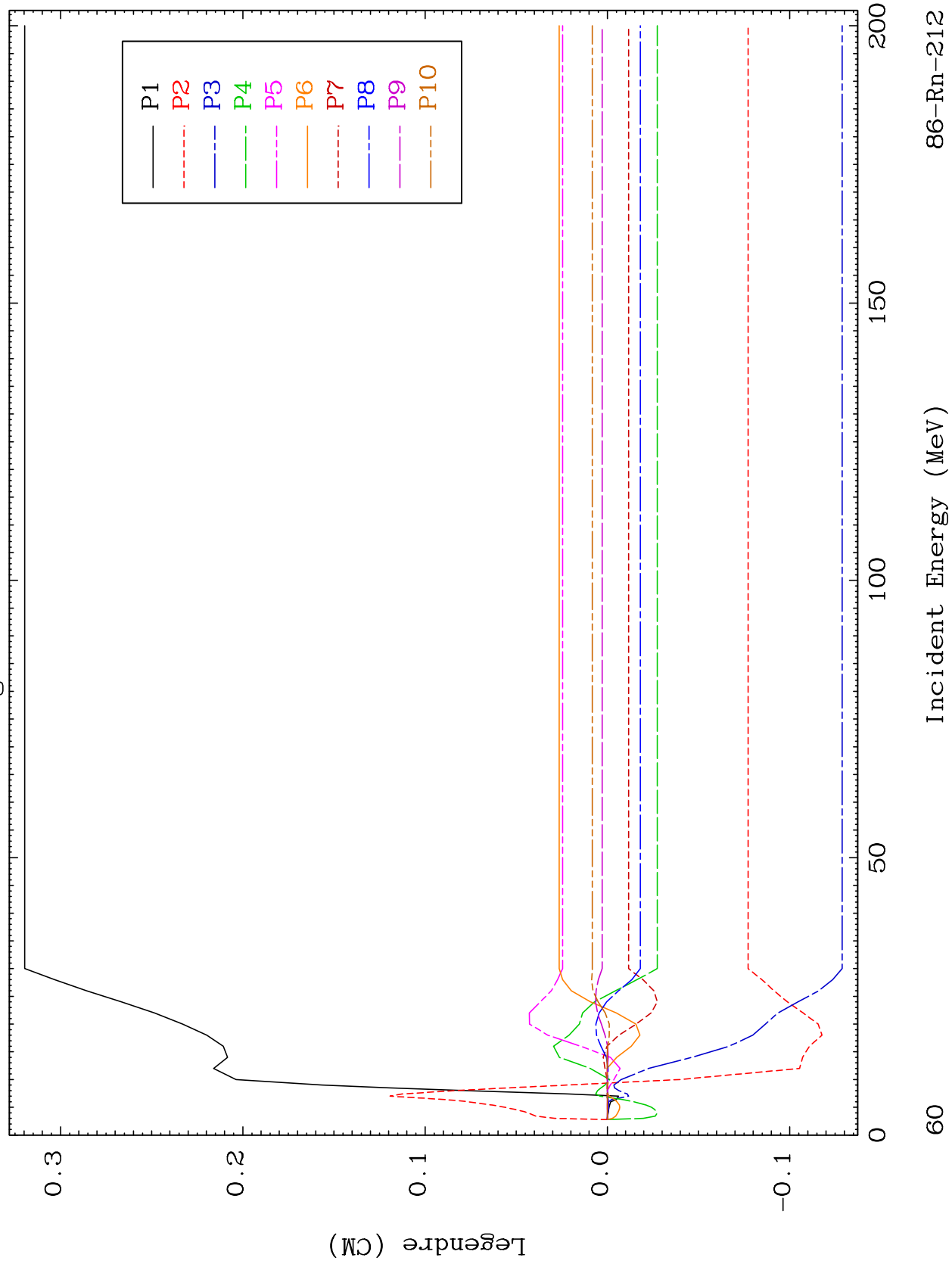




MAT 8628

MT= 61 (n, n') Level
Legendre Coefficients

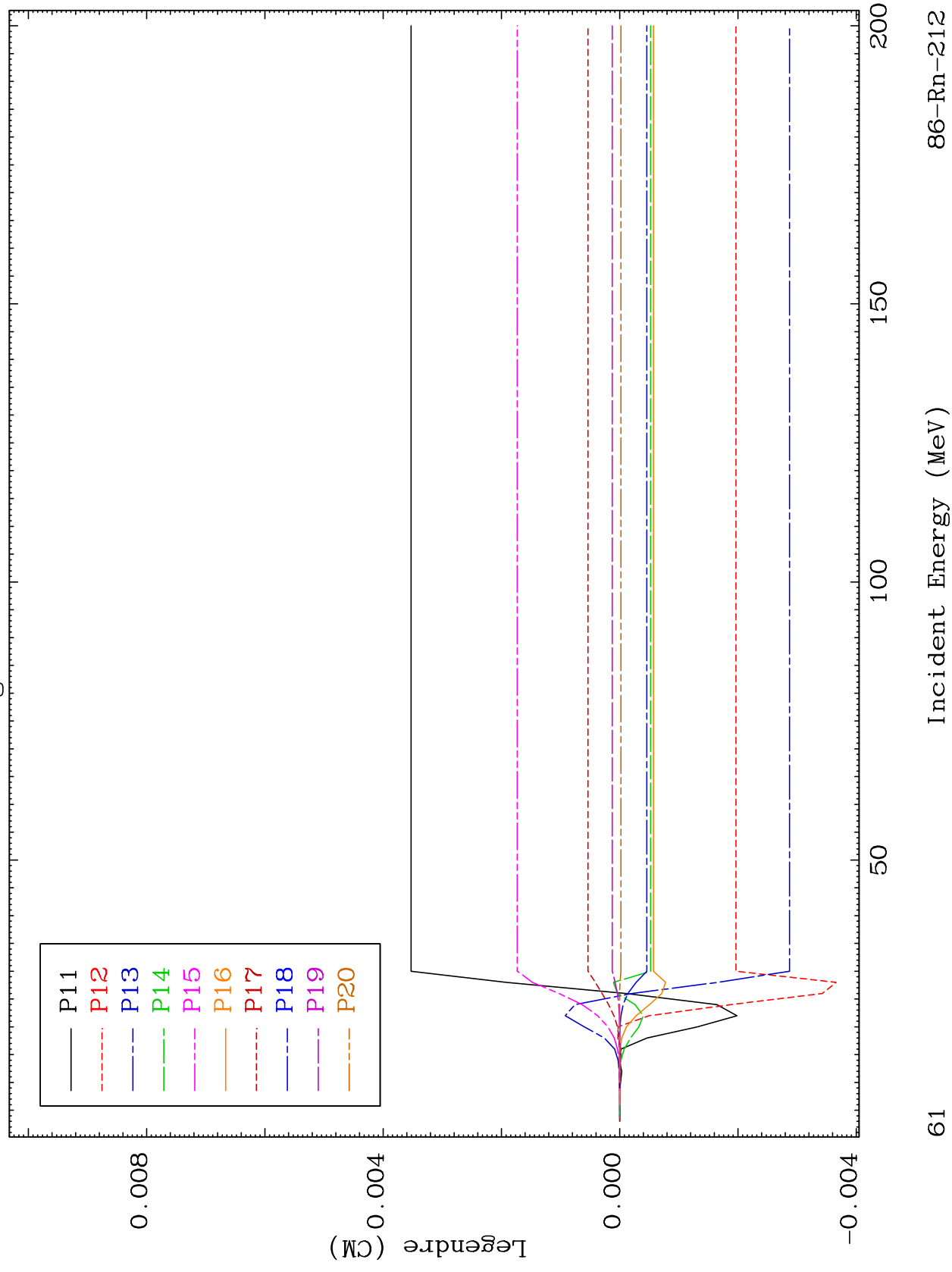
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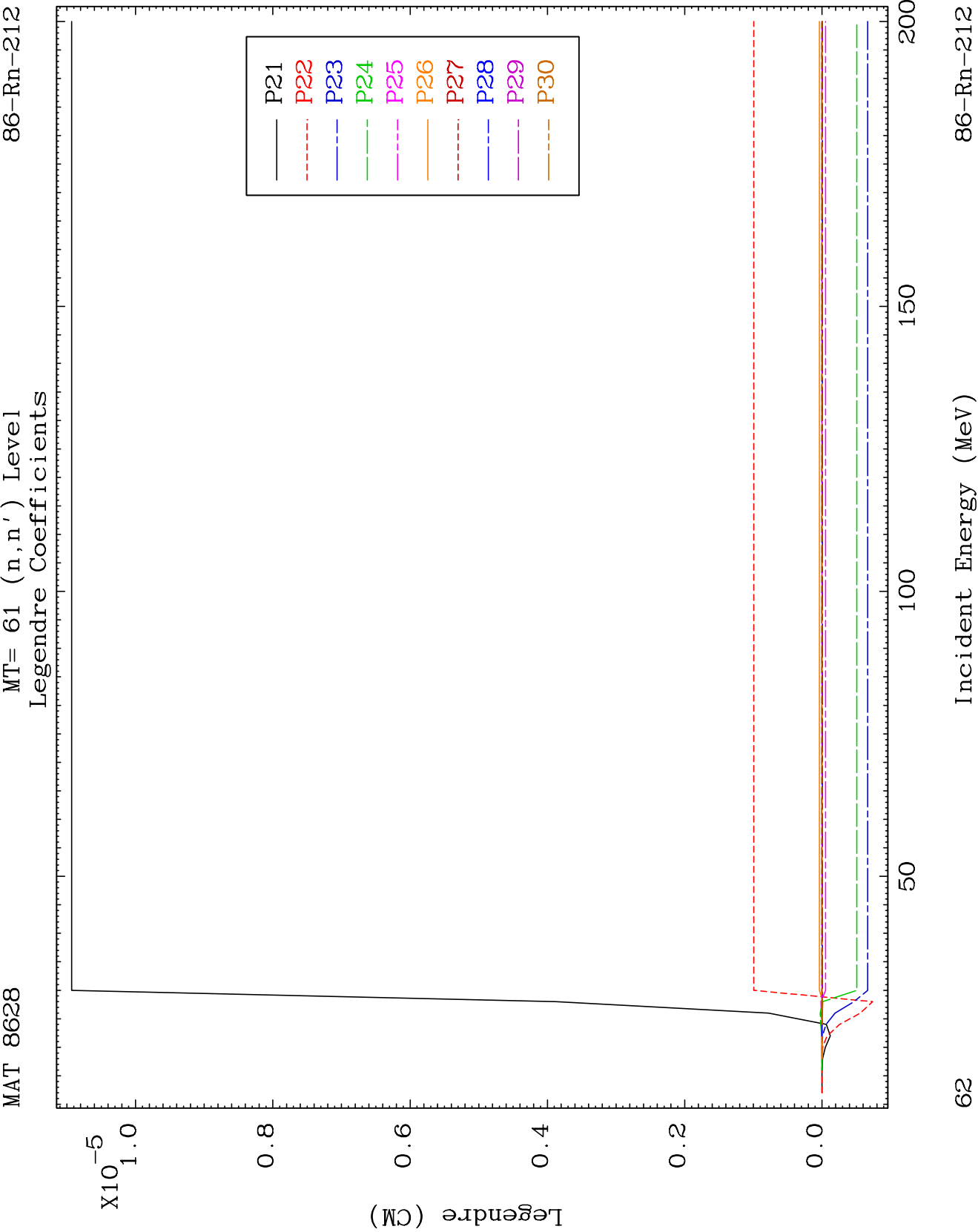


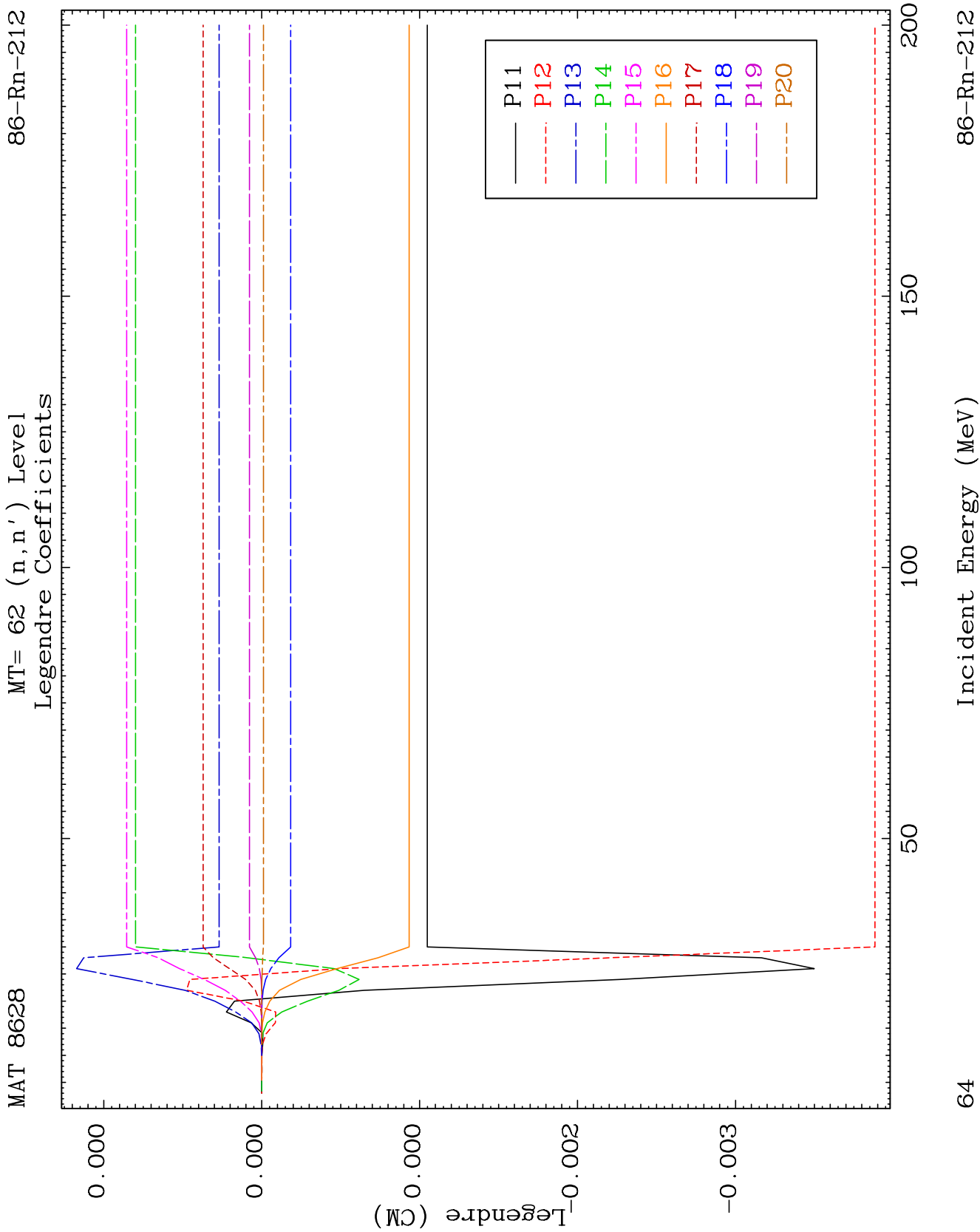
MAT 8628

MT= 61 (n, n') Level
Legendre Coefficients

86-Rn-212



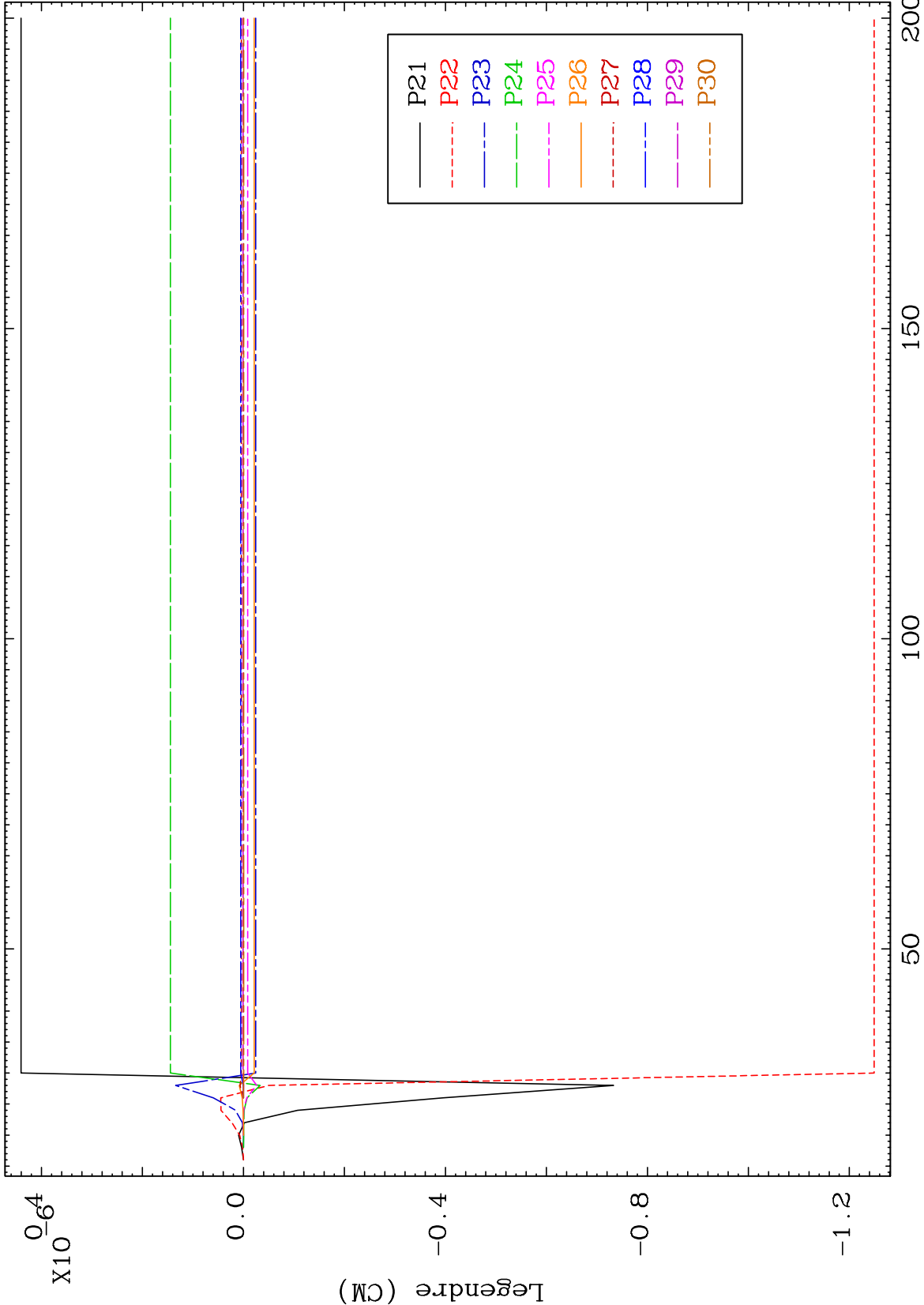




MAT 8628

MT= 62 (n,n') Level
Legendre Coefficients

86-Rn-212



65

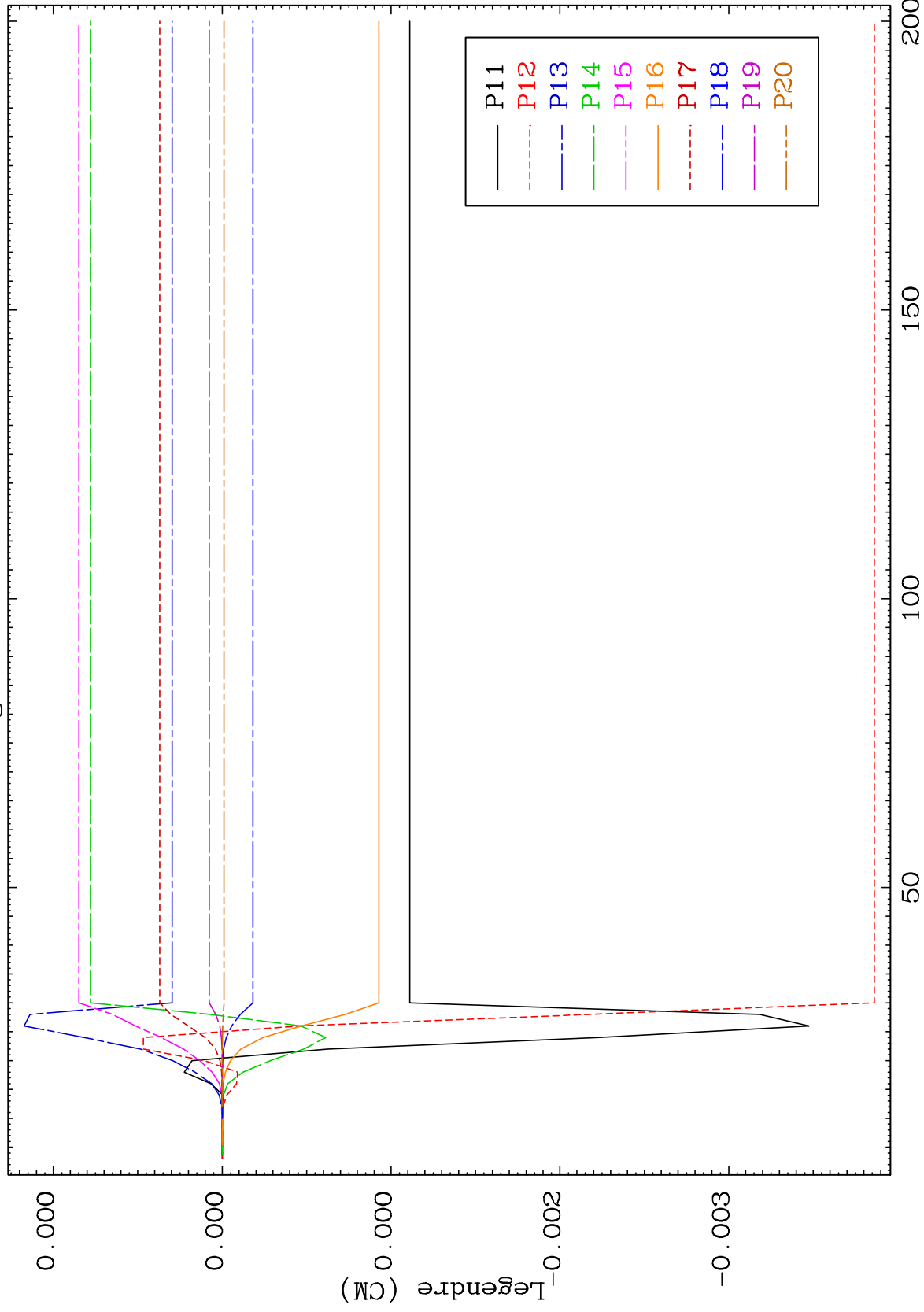
Incident Energy (MeV)

86-Rn-212

MAT 8628

MT= 63 (n,n') Level
Legendre Coefficients

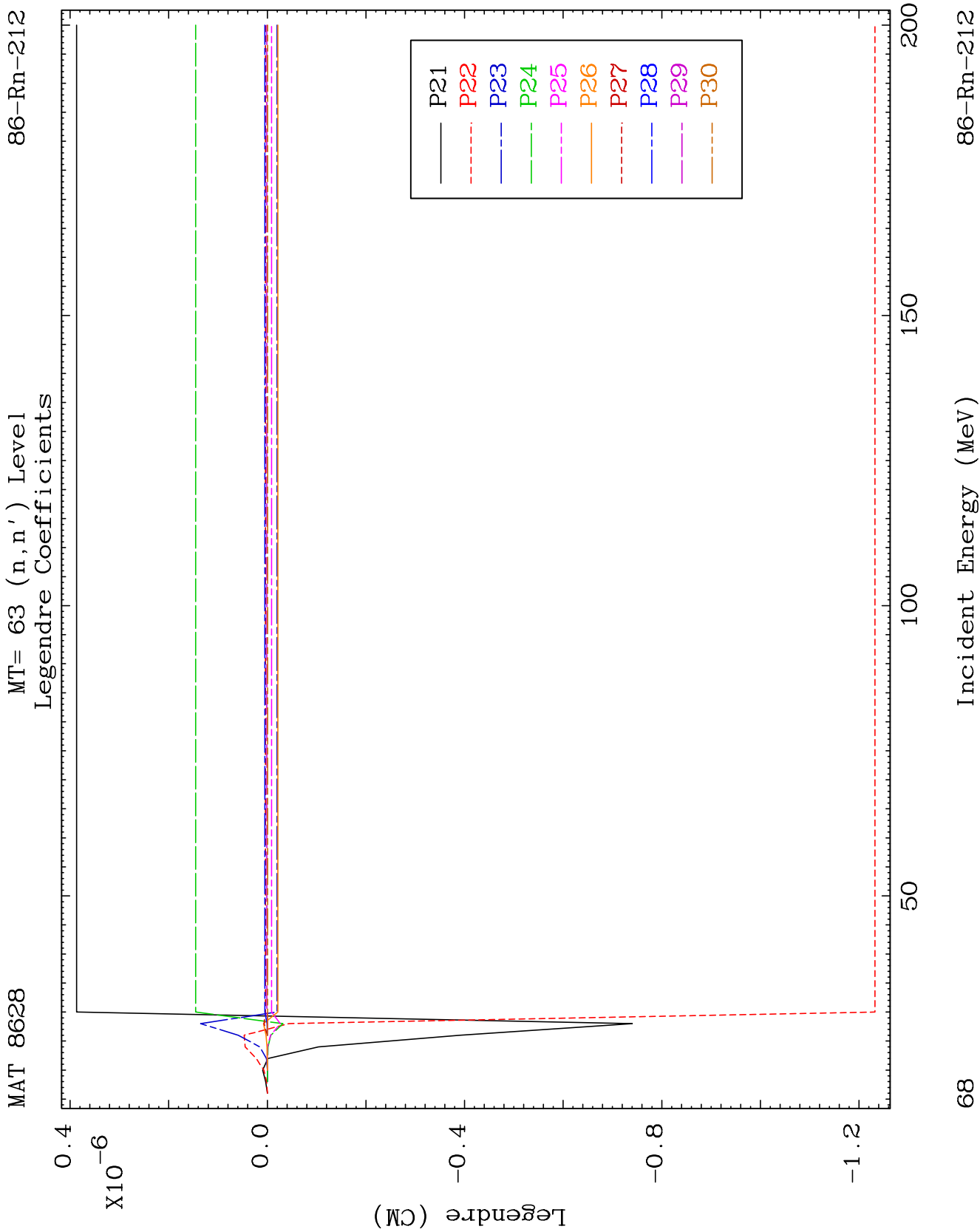
86-Rn-212

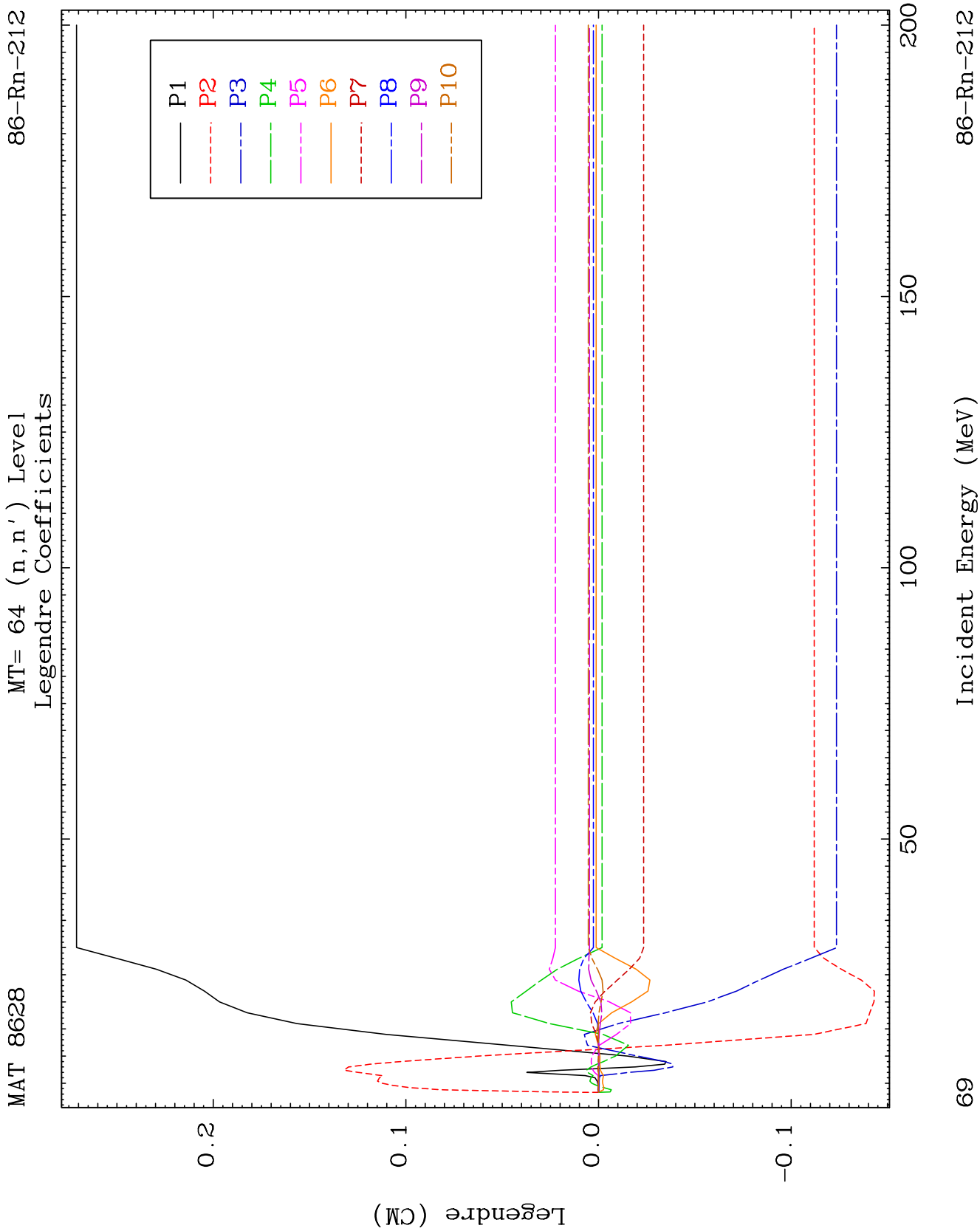


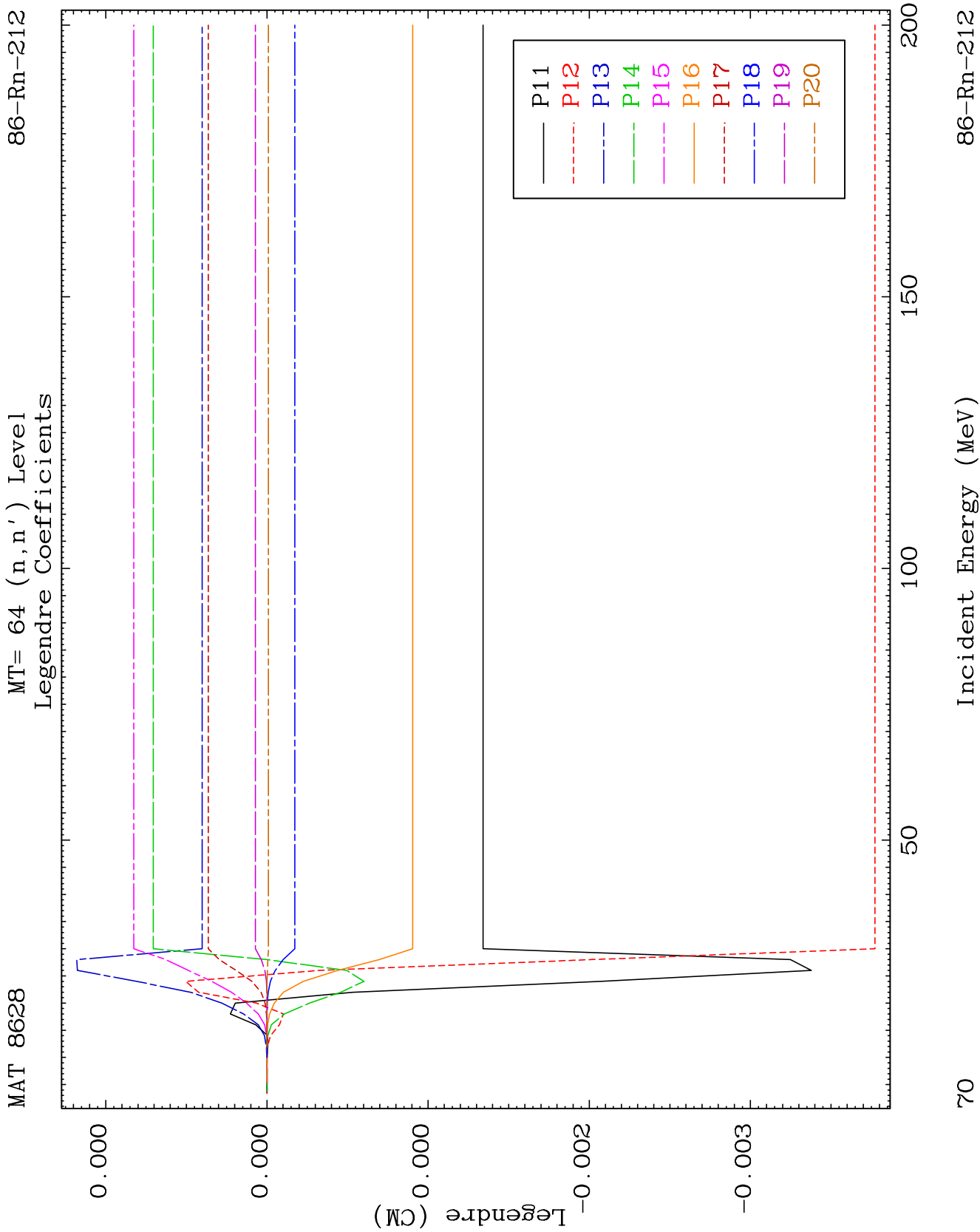
67

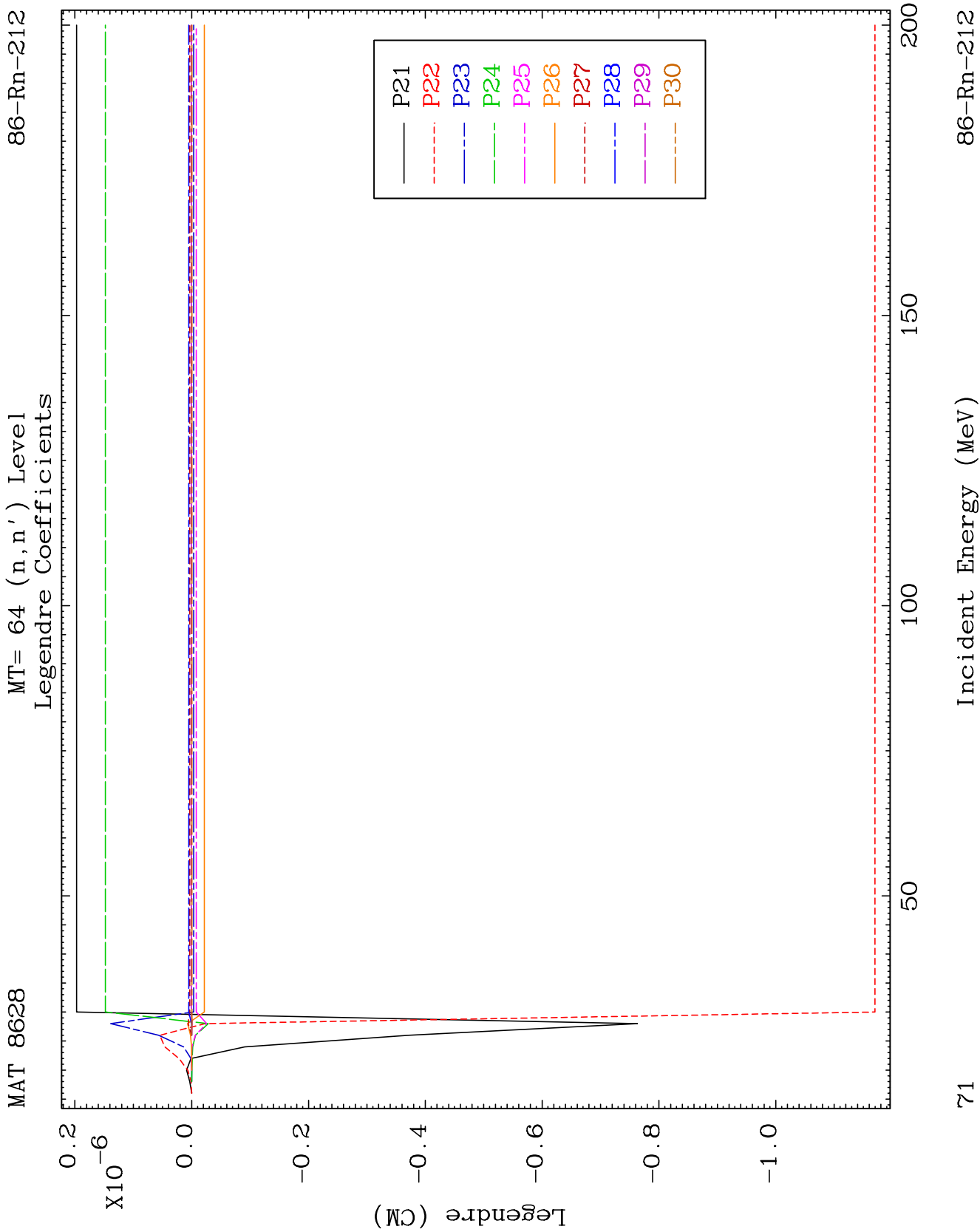
Incident Energy (MeV)

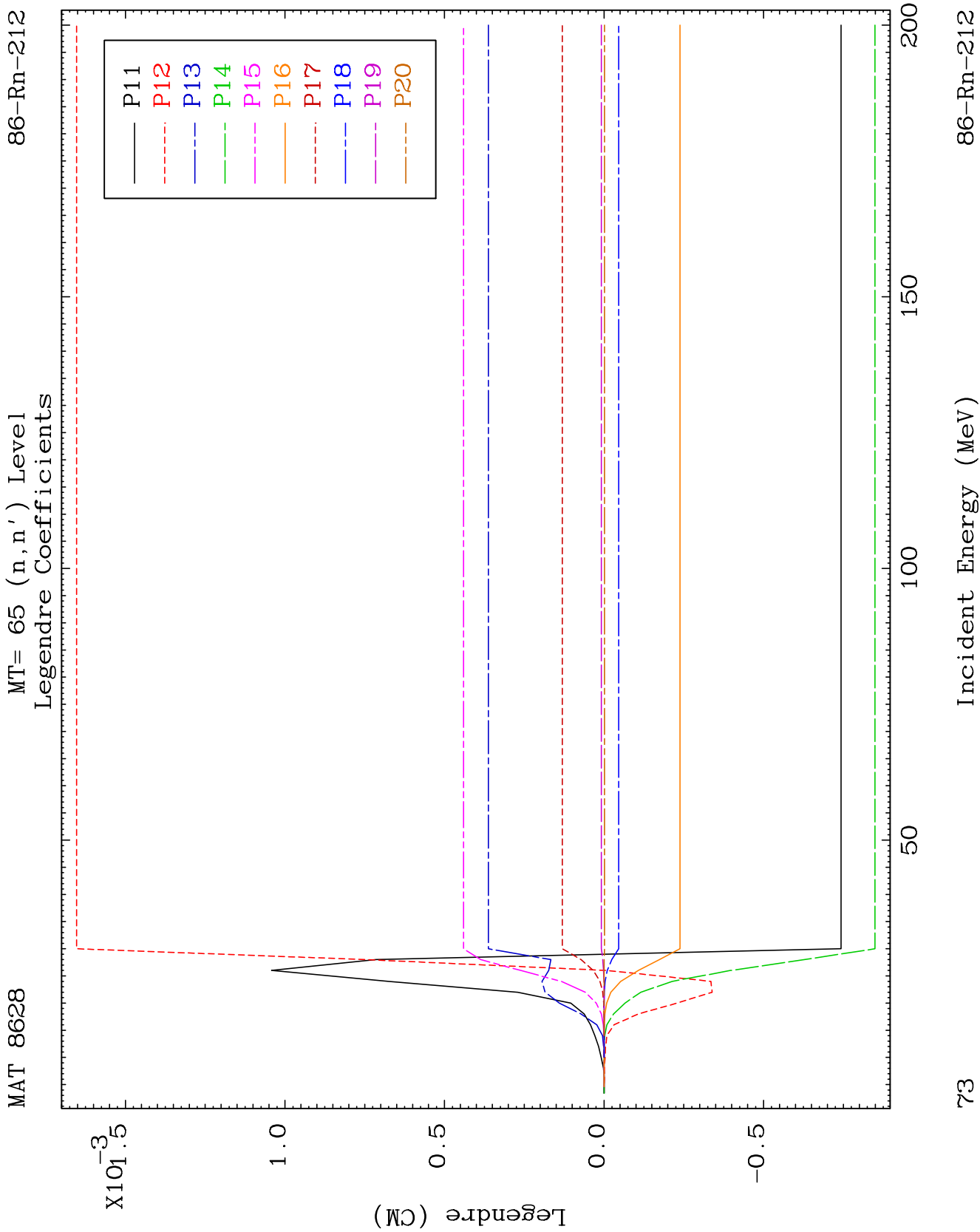
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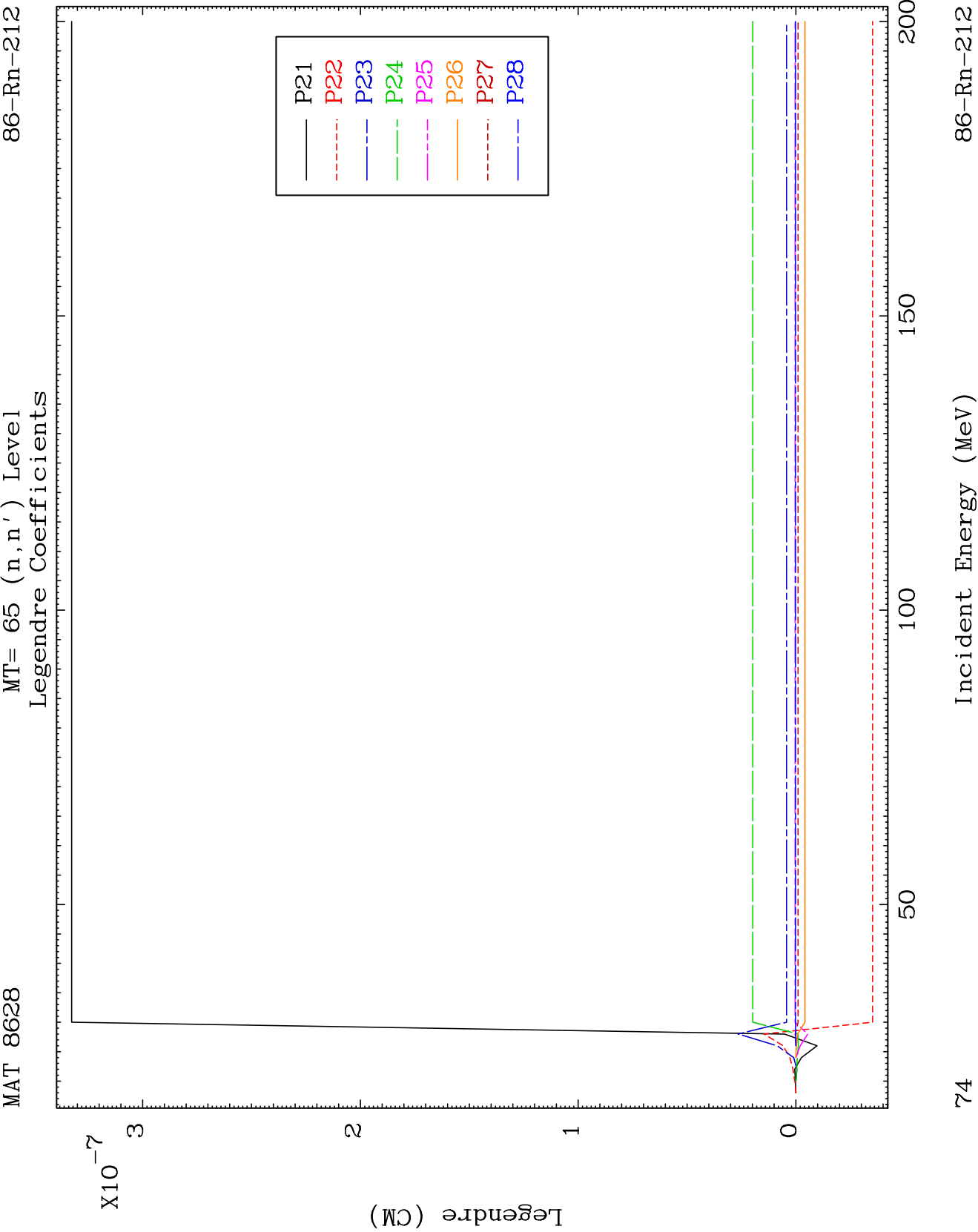


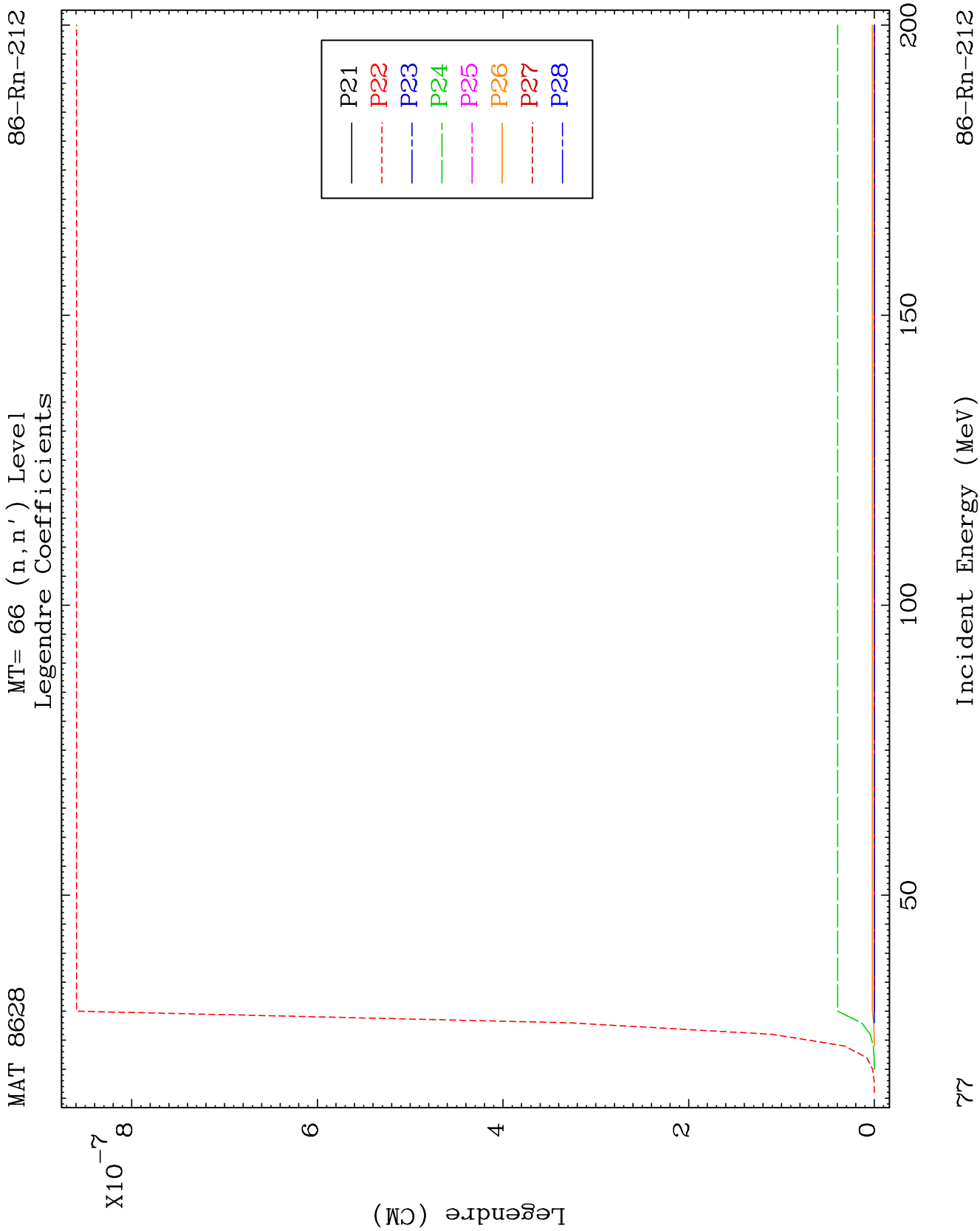








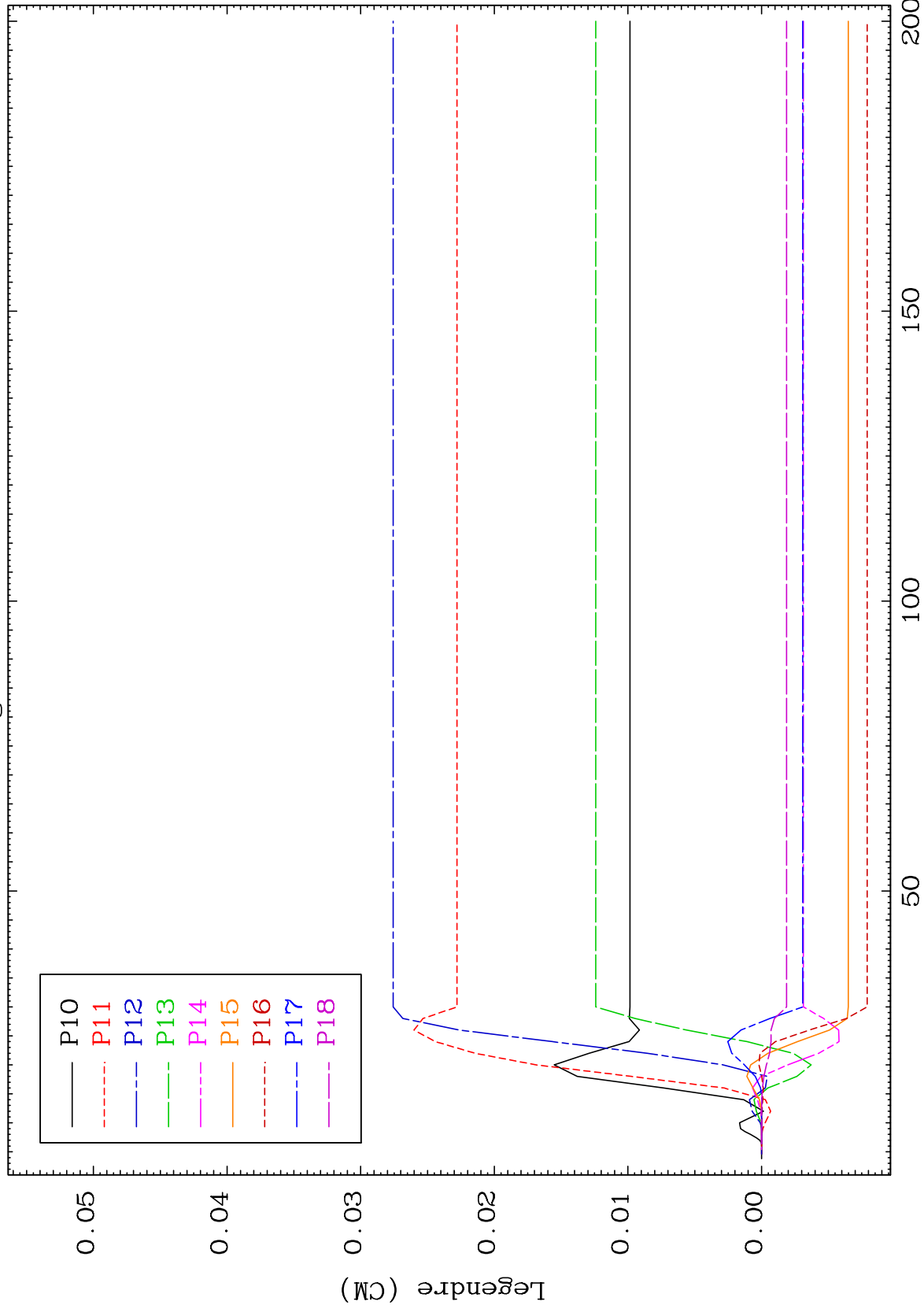




MAT 8628

MT= 67 (n,n') Level
Legendre Coefficients

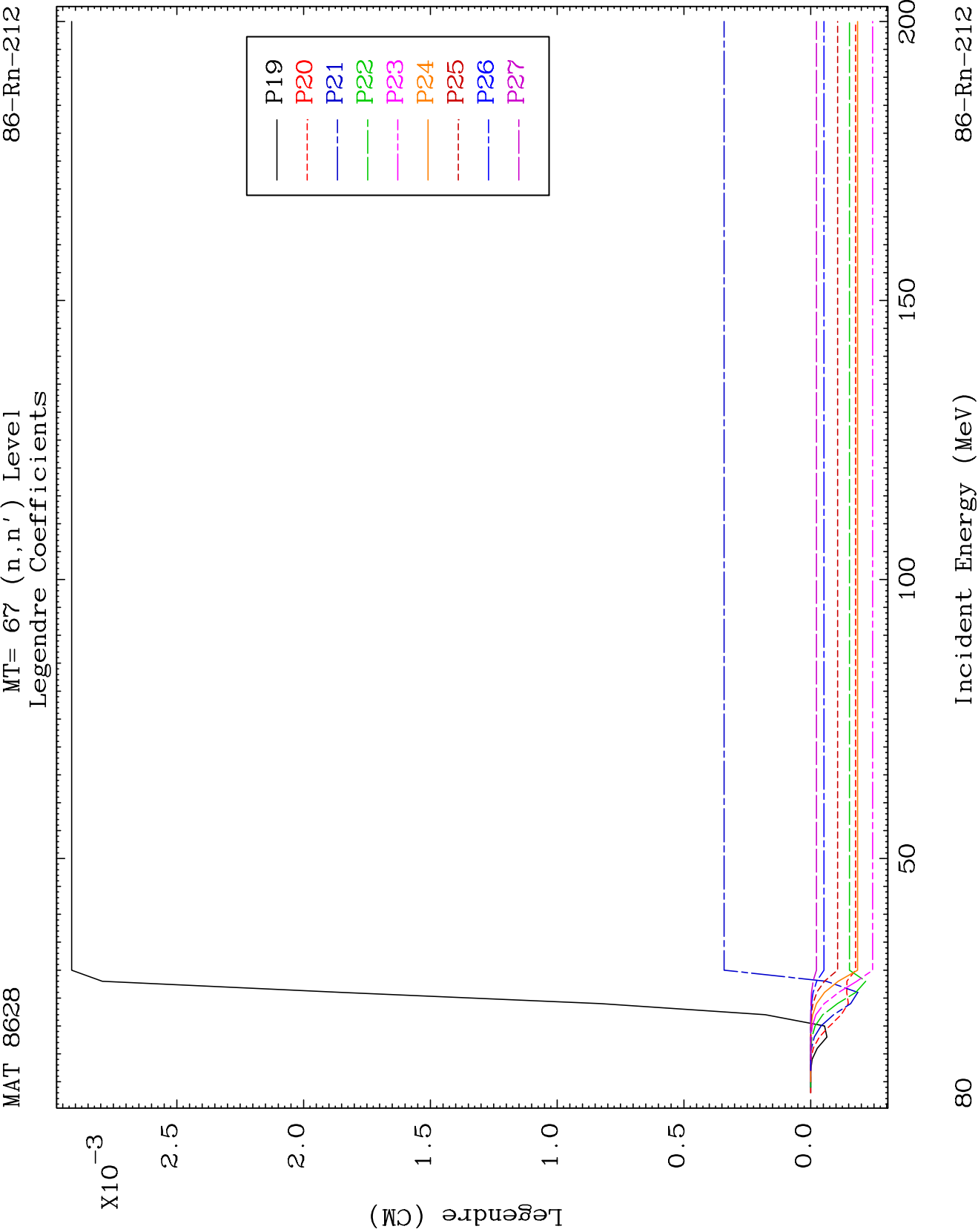
86-Rn-212

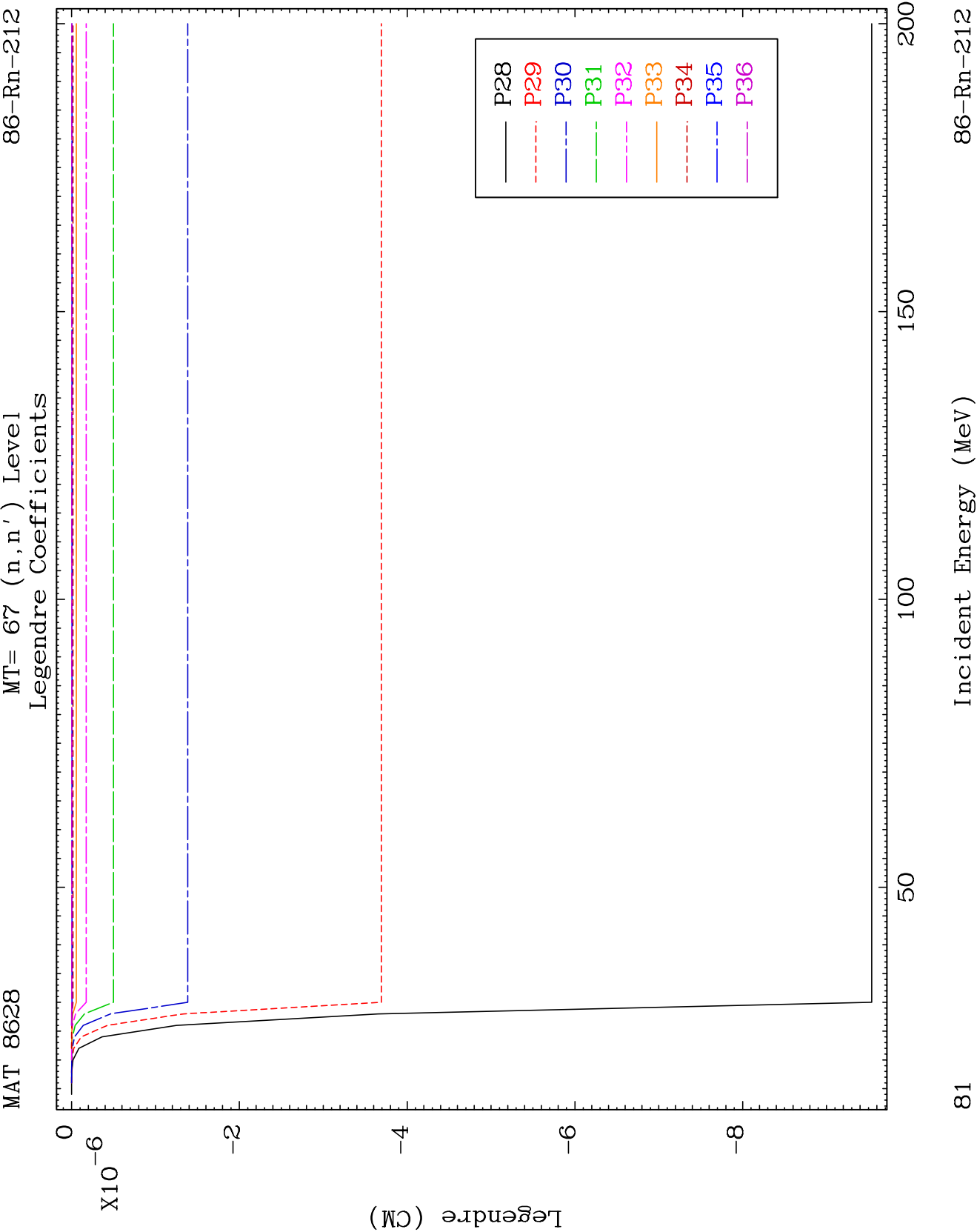


79

Incident Energy (MeV)

86-Rn-212

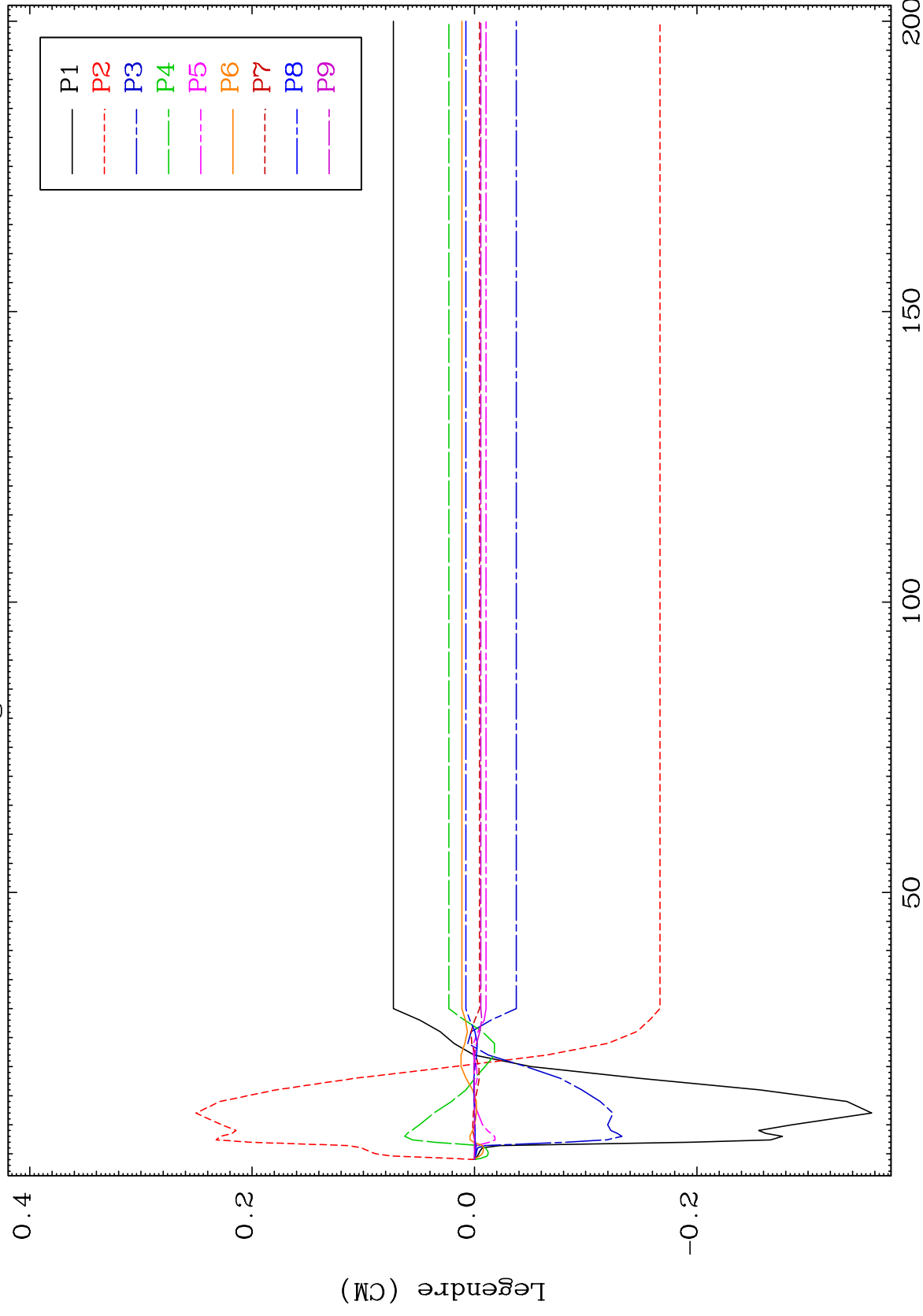




MAT 8628

MT= 68 (n, n') Level
Legendre Coefficients

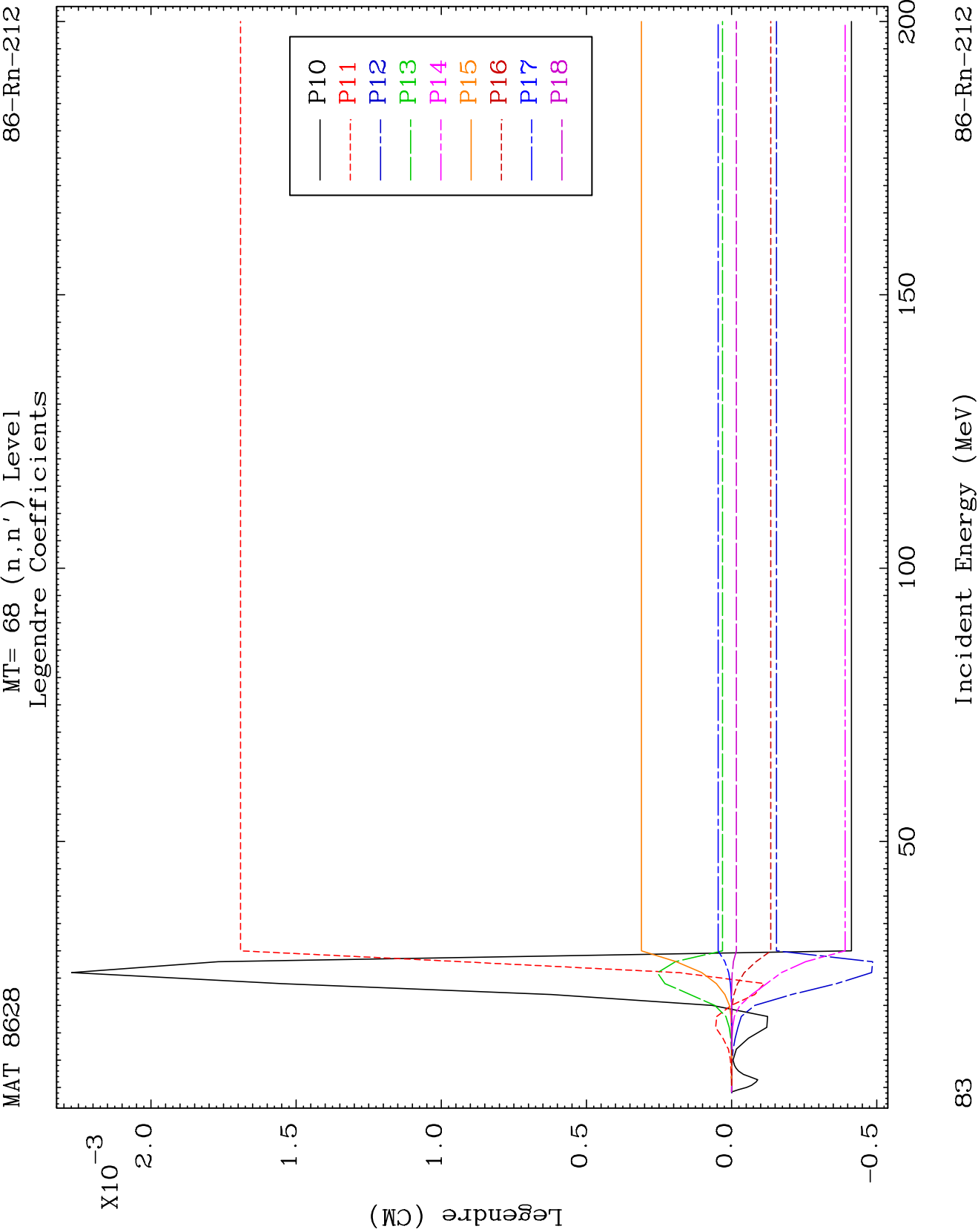
86-Rn-212

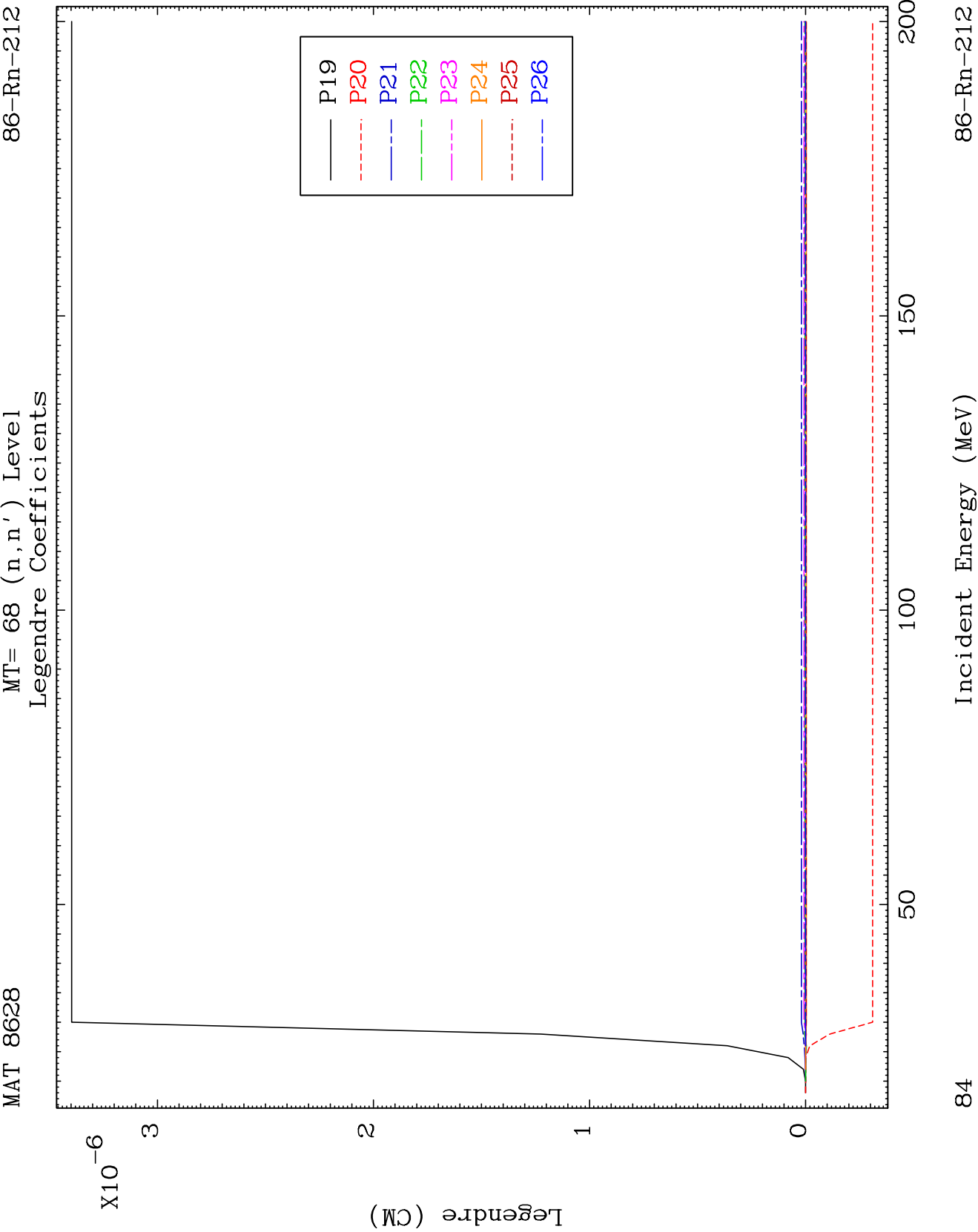


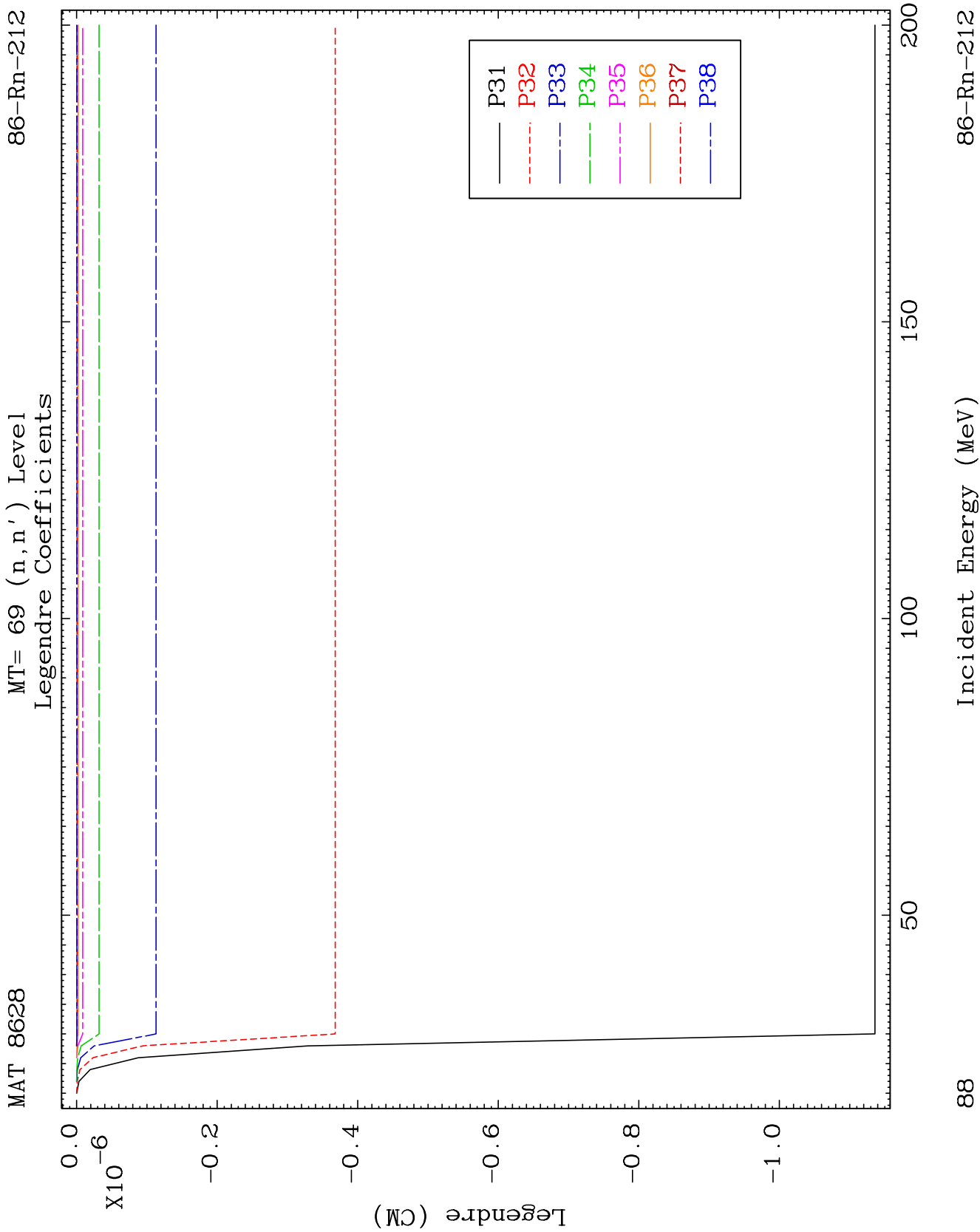
28

Incident Energy (MeV)

86-Rn-212



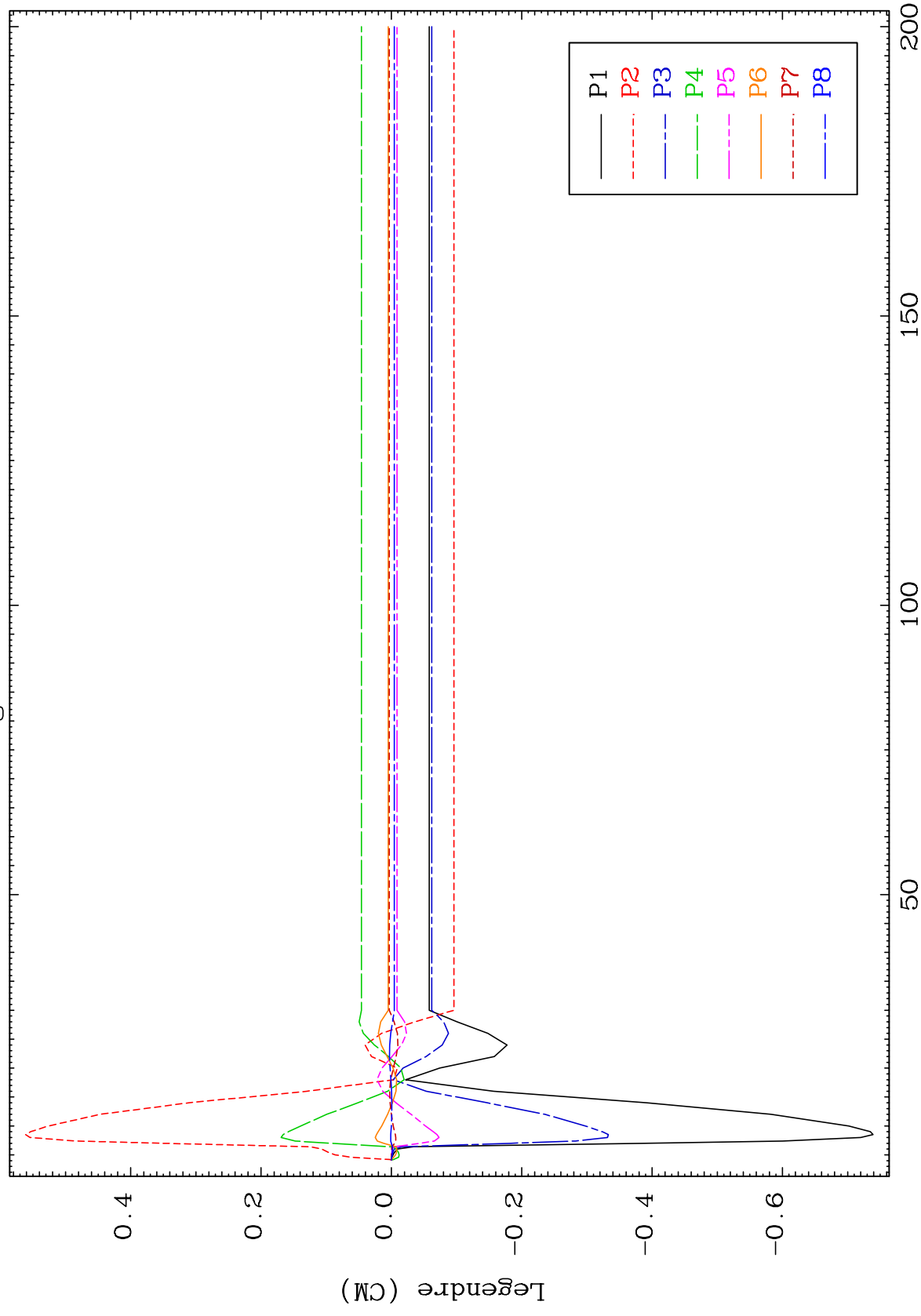




MAT 8628

86-Rn-212

MT= 70 (n, n') Level
Legendre Coefficients



98

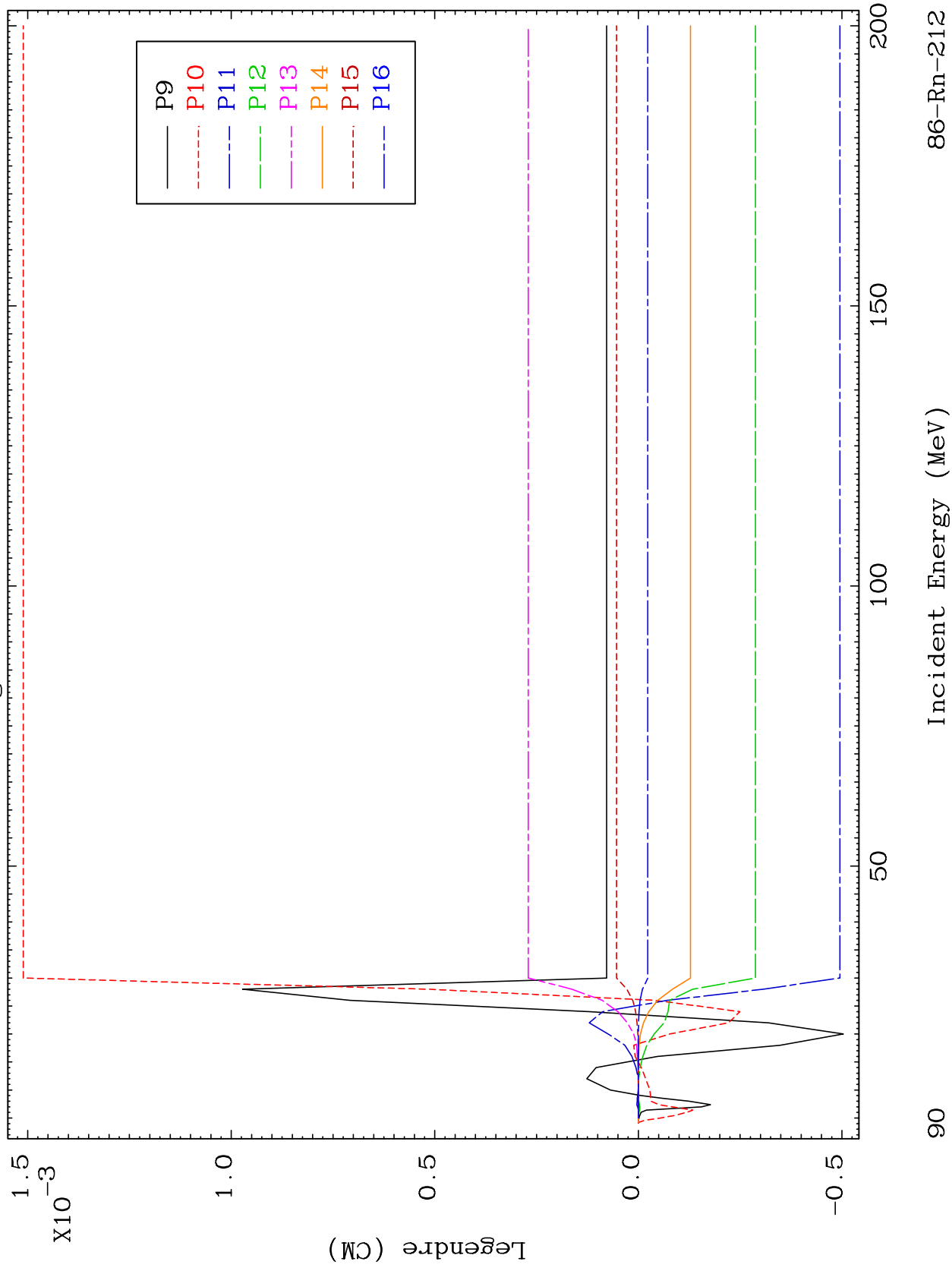
Incident Energy (MeV)

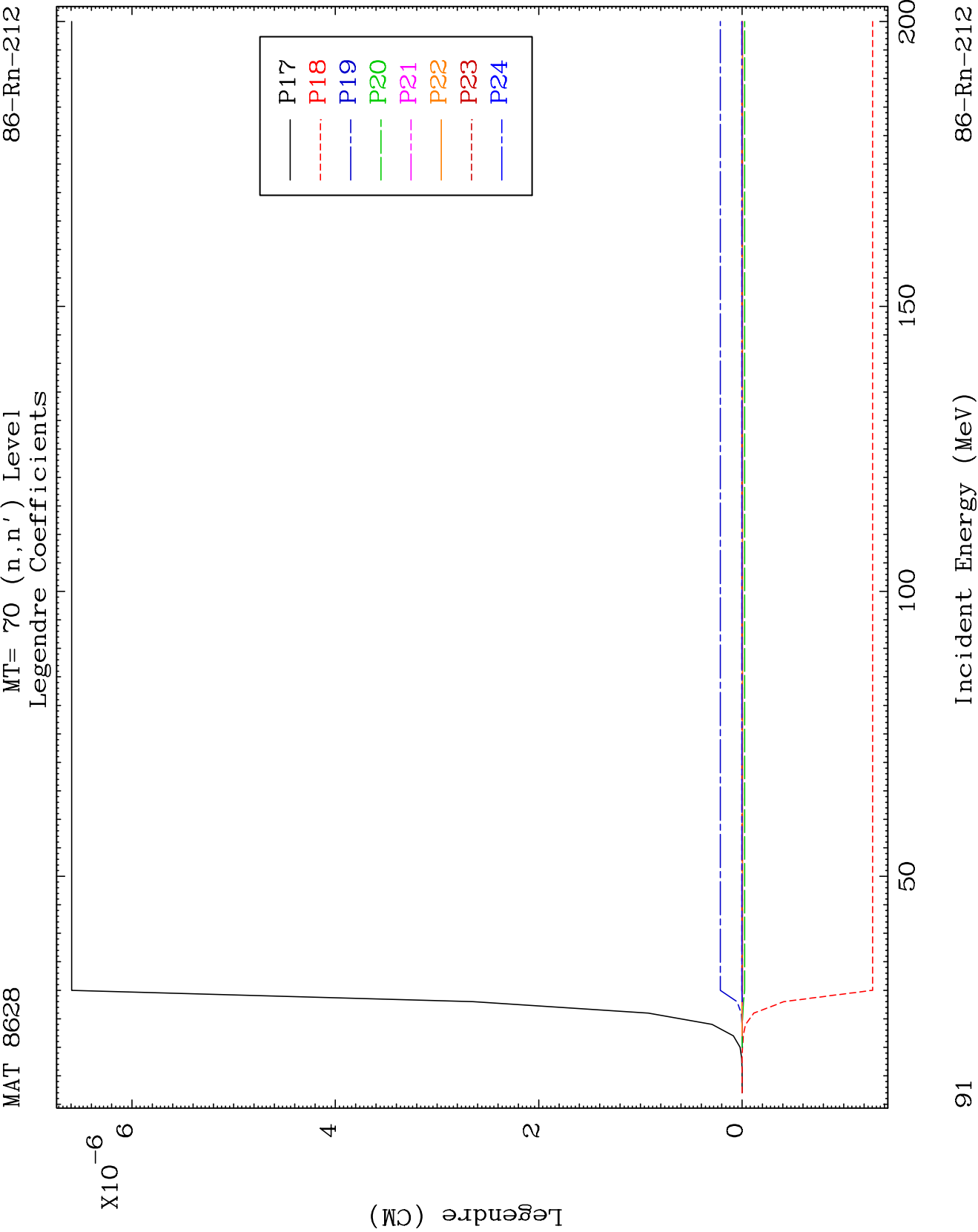
86-Rn-212

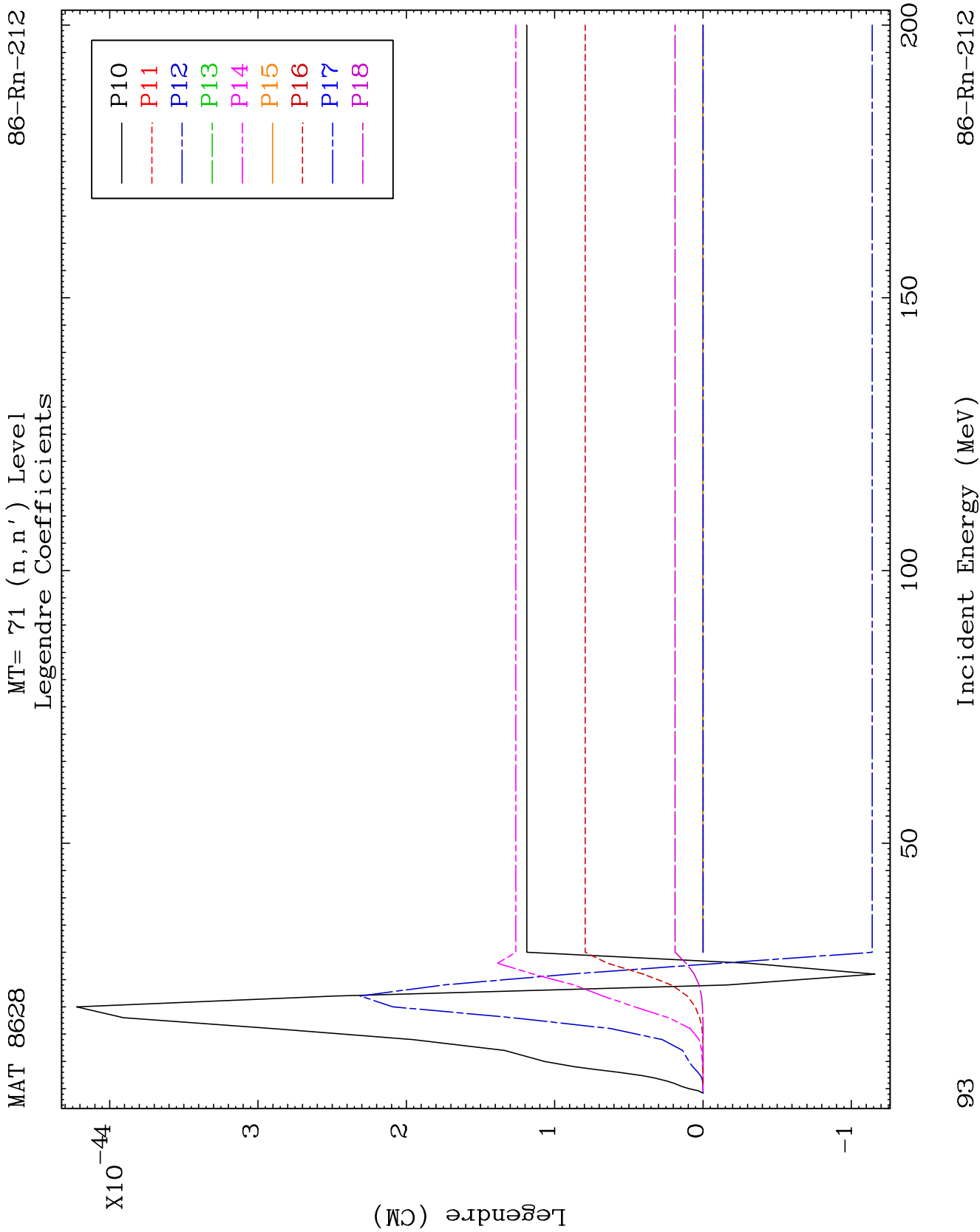
MAT 8628

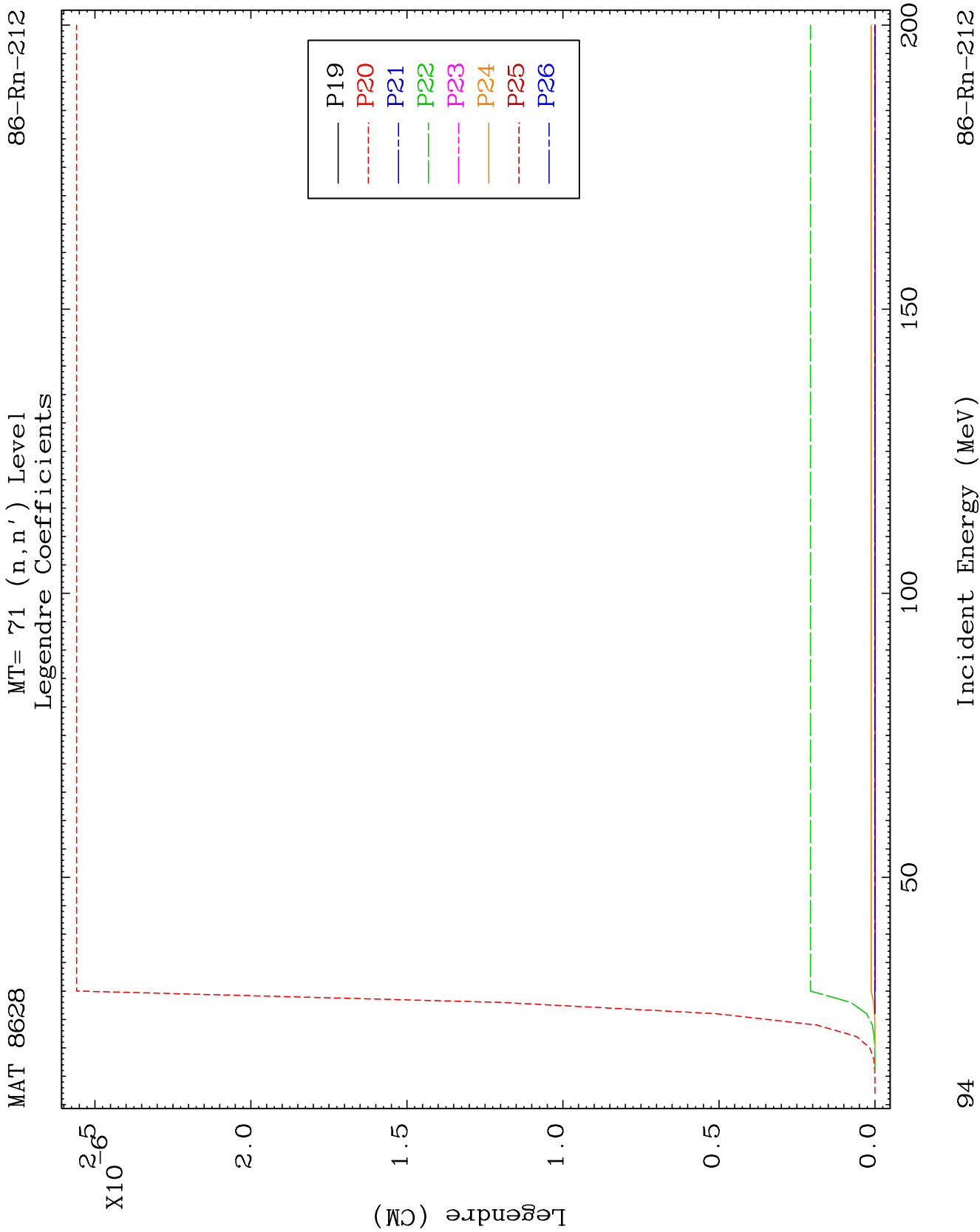
MT= 70 (n, n') Level
Legendre Coefficients

86-Rn-212





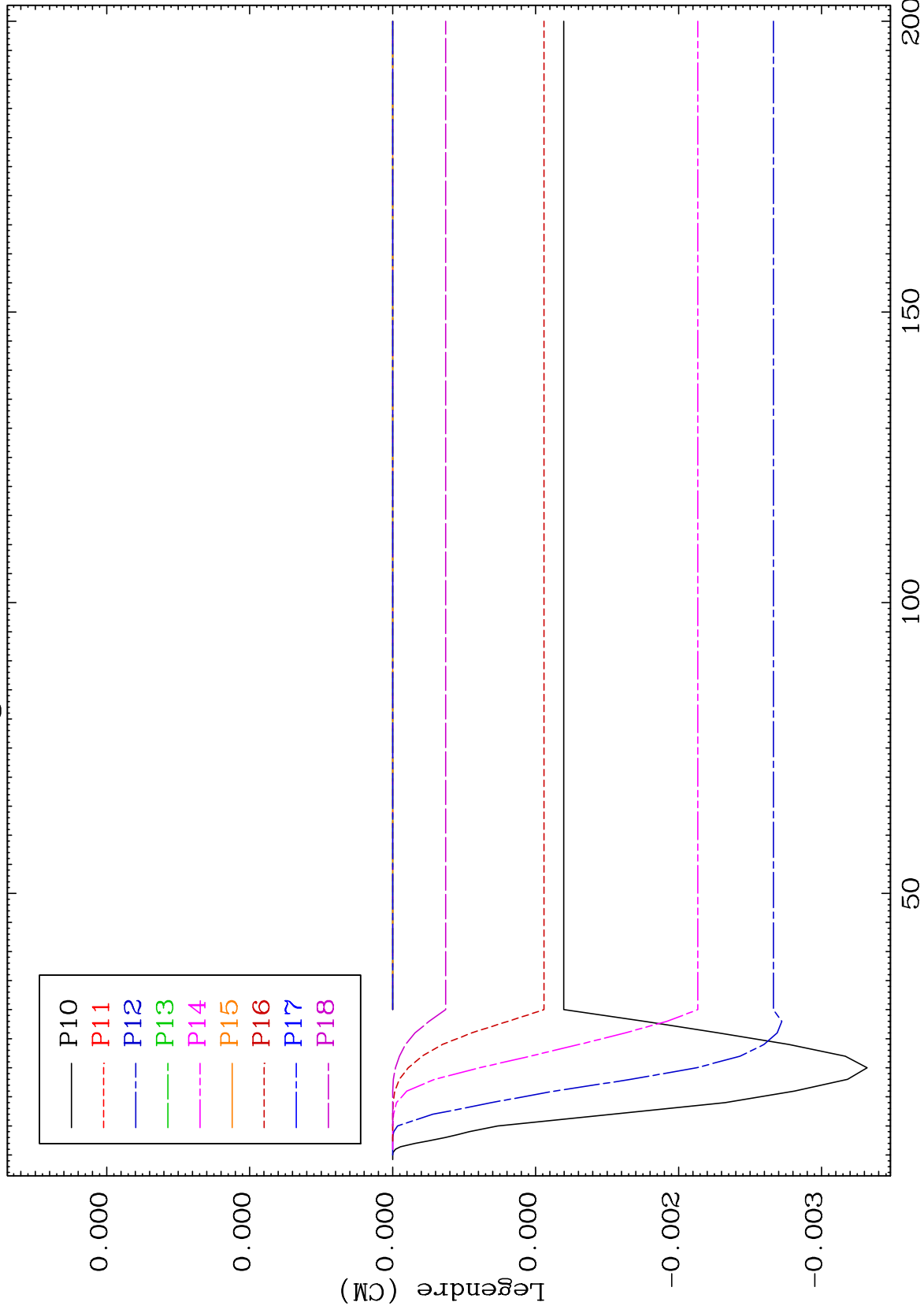




MAT 8628

MT= 72 (n,n') Level
Legendre Coefficients

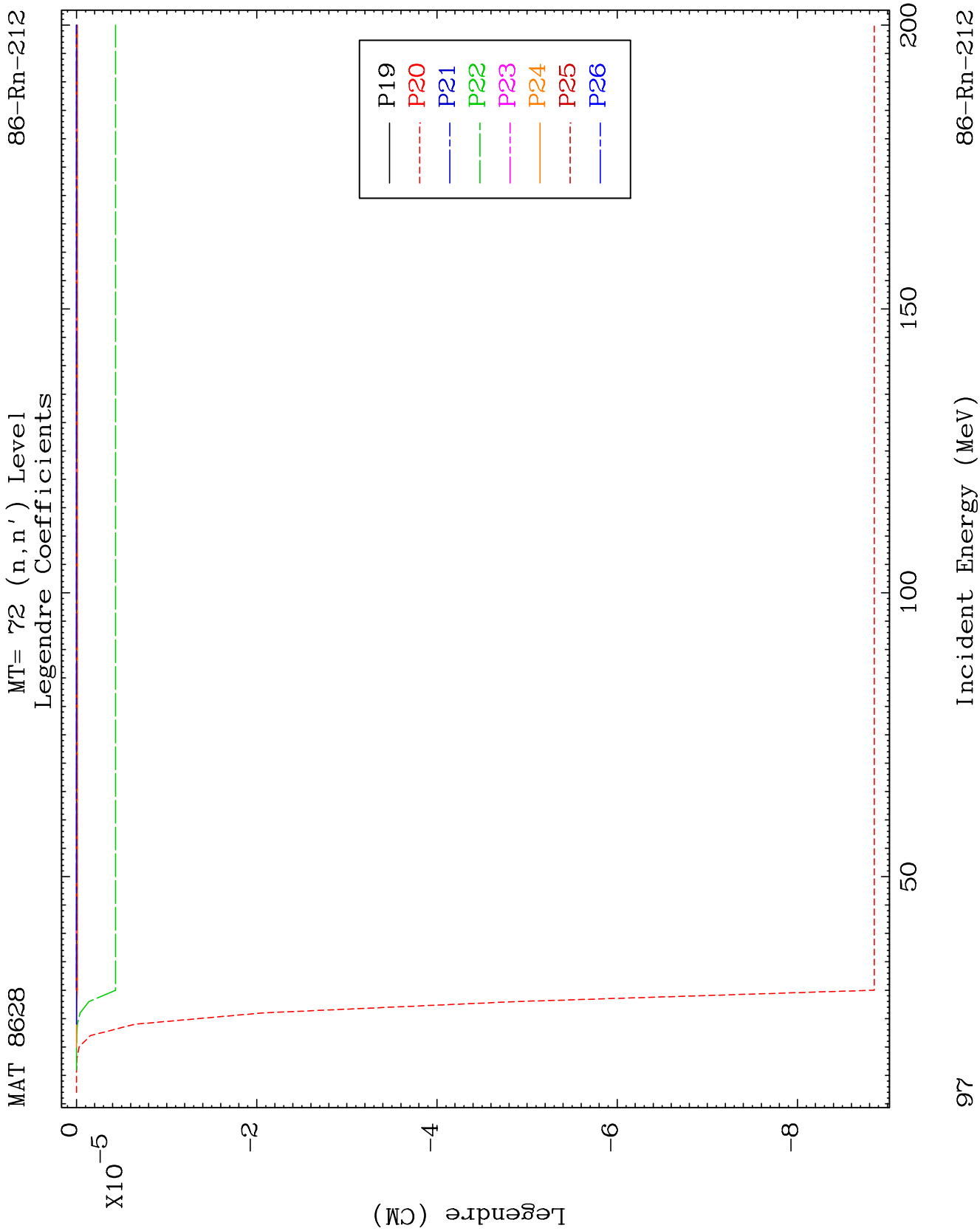
86-Rn-212



96

Incident Energy (MeV)

86-Rn-212

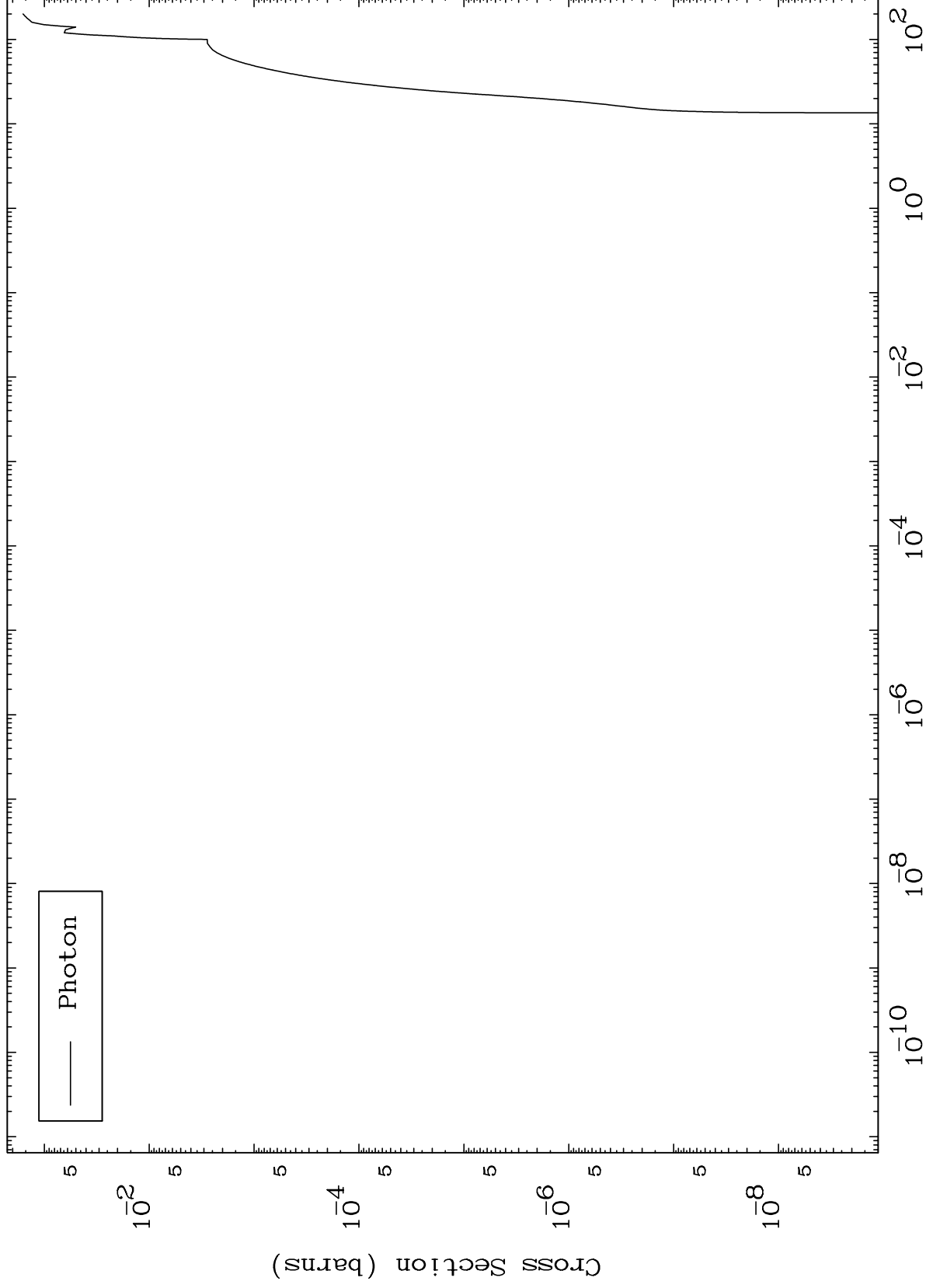


MAT 8628

Fission

86-Rn-212

Radionuclide Production Cross Section



98

Incident Energy (MeV)

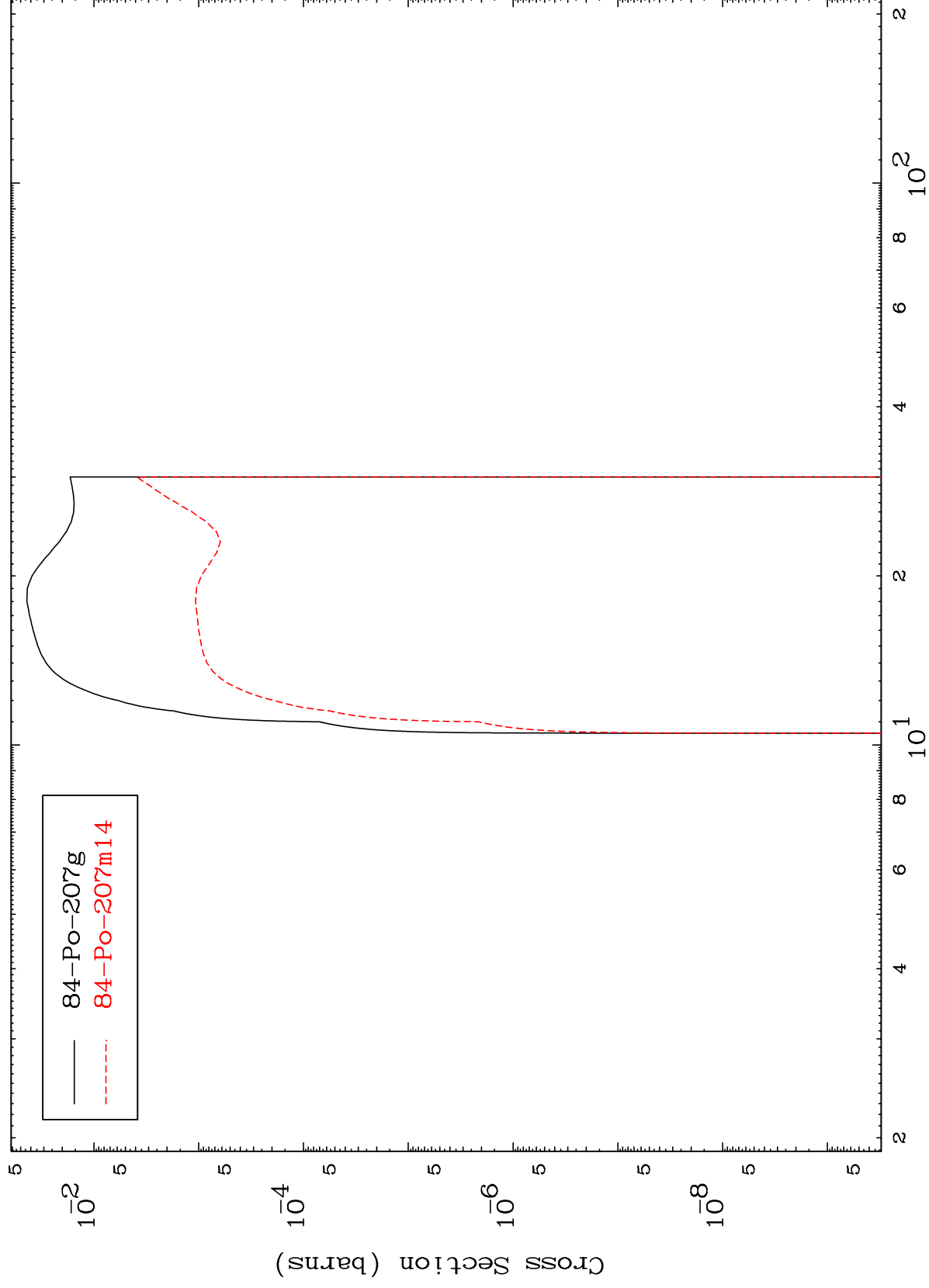
86-Rn-212

MAT 8628

86-Rn-212

Radionuclide Production Cross Section

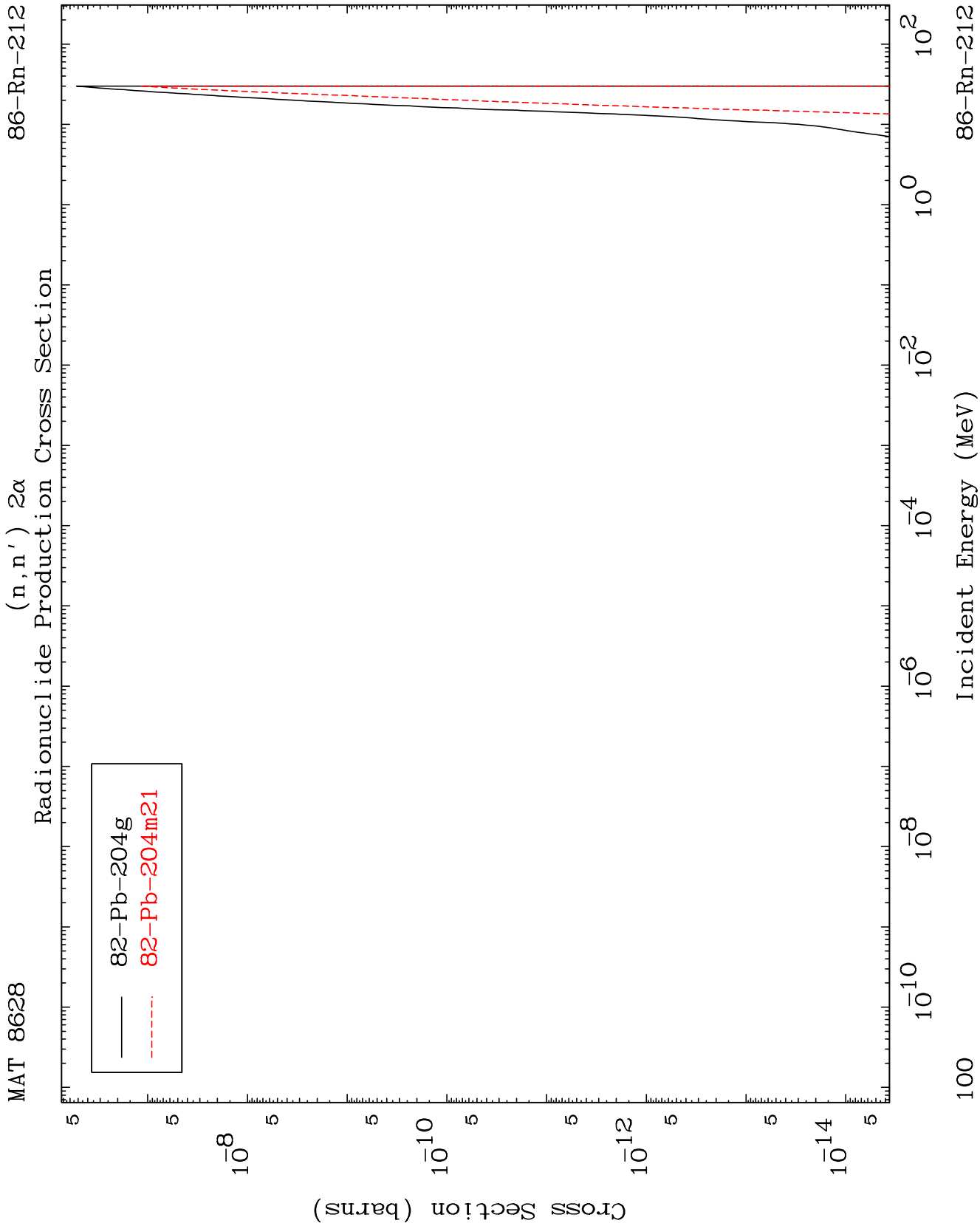
(n,2n) α

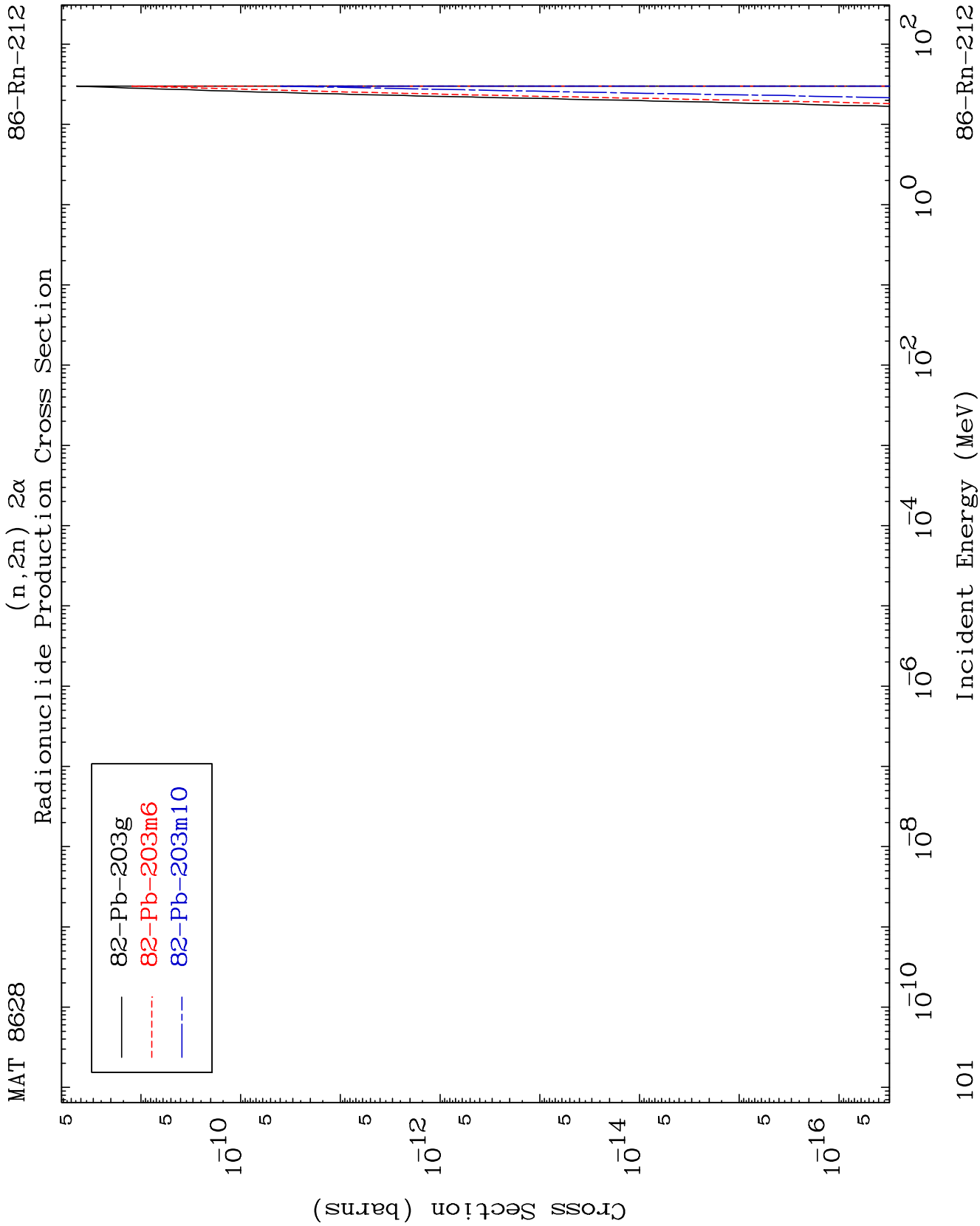


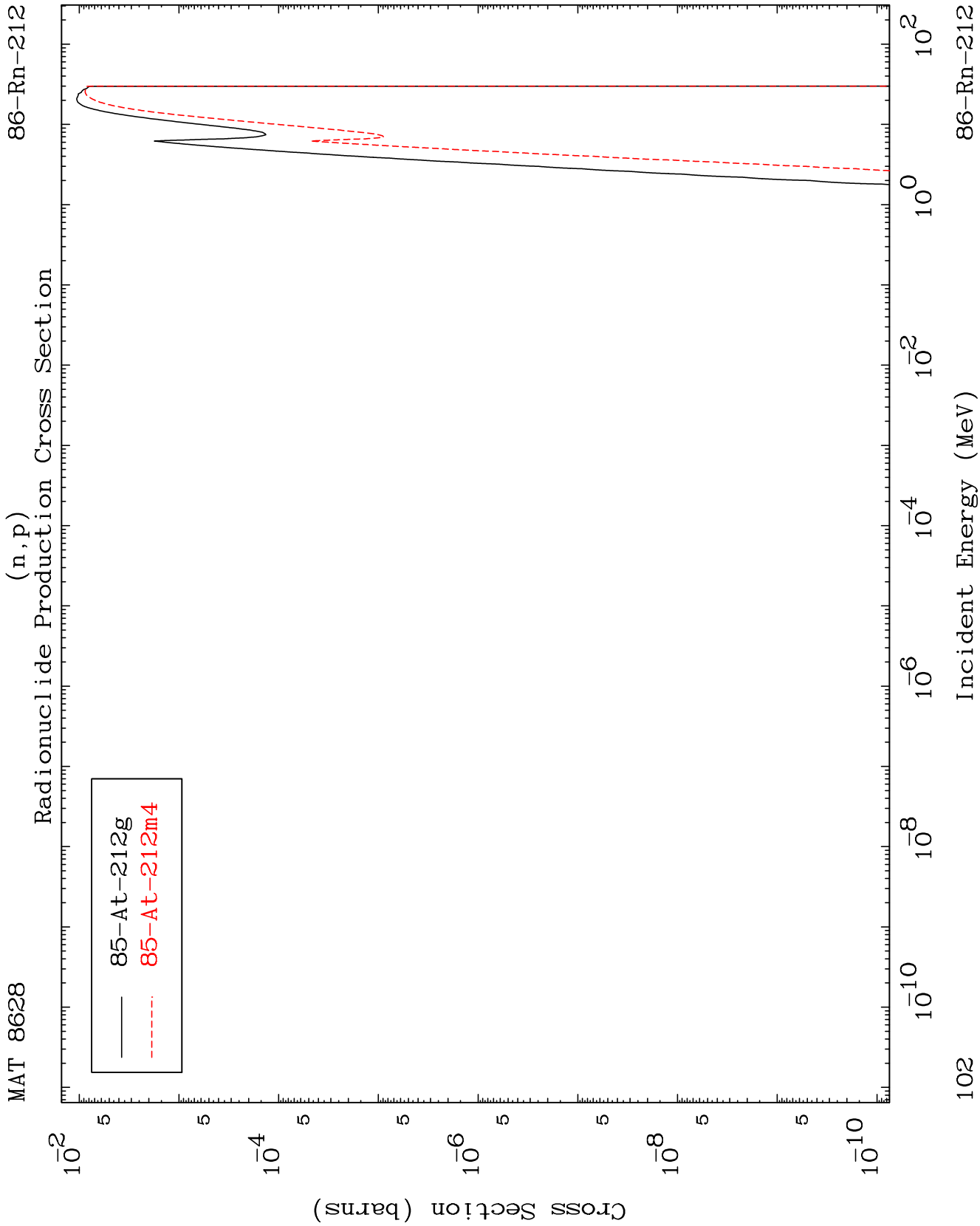
99

Incident Energy (MeV)

86-Rn-212



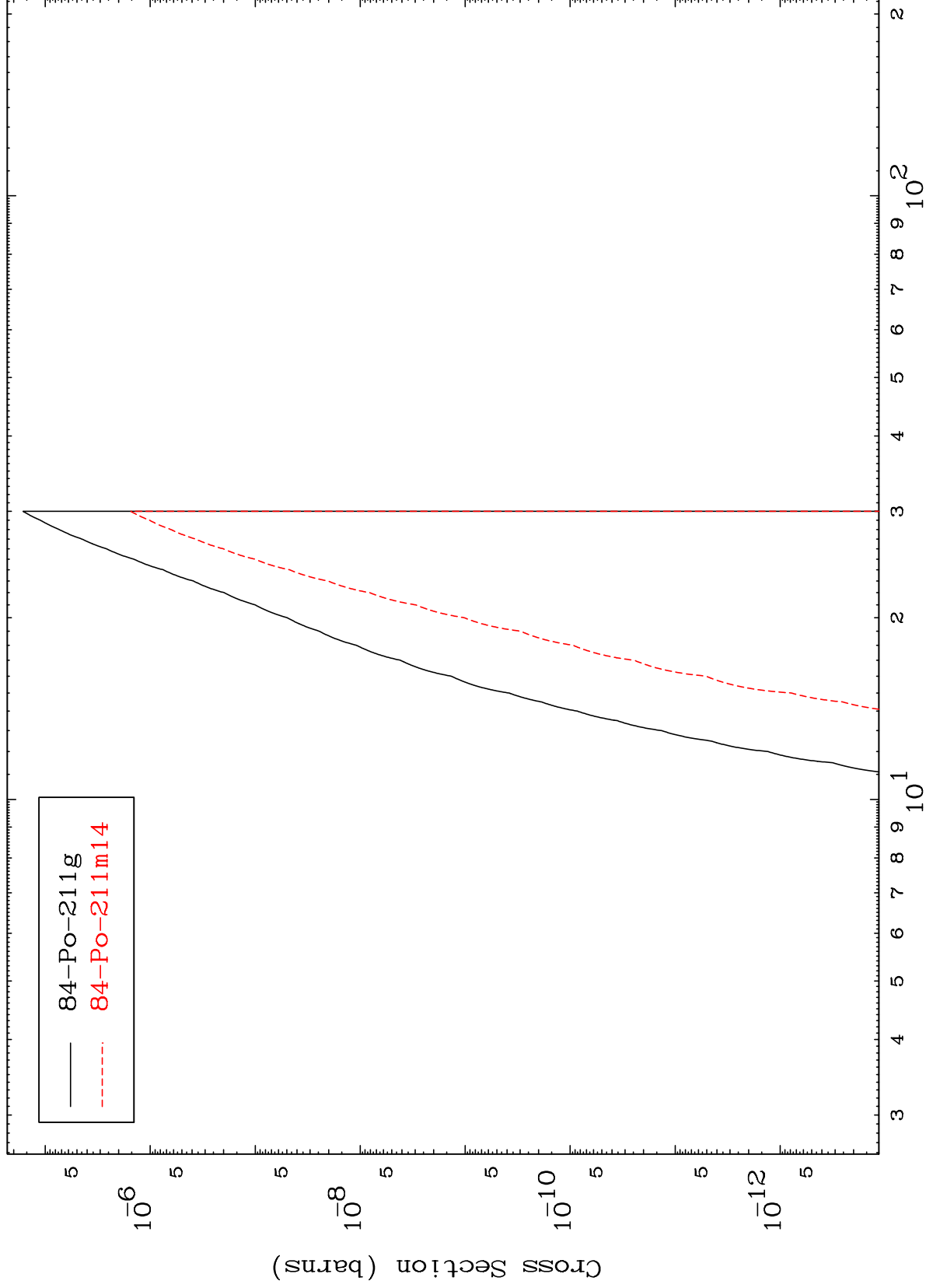




MAT 8628

86-Rn-212

(n,2p)
Radionuclide Production Cross Section



103

Incident Energy (MeV)

86-Rn-212