

Program EVALPLOT
(Version 2017-1)

by

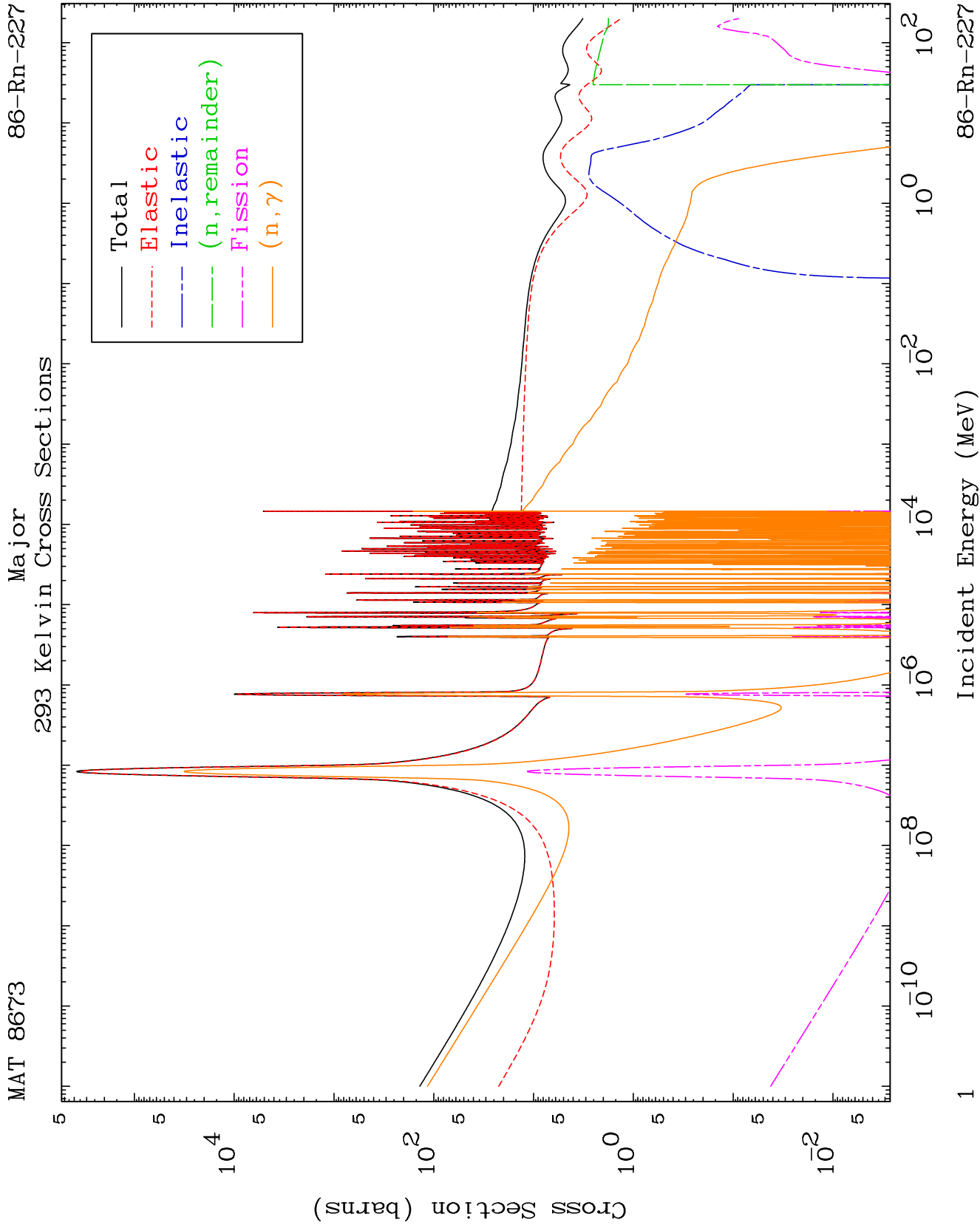
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
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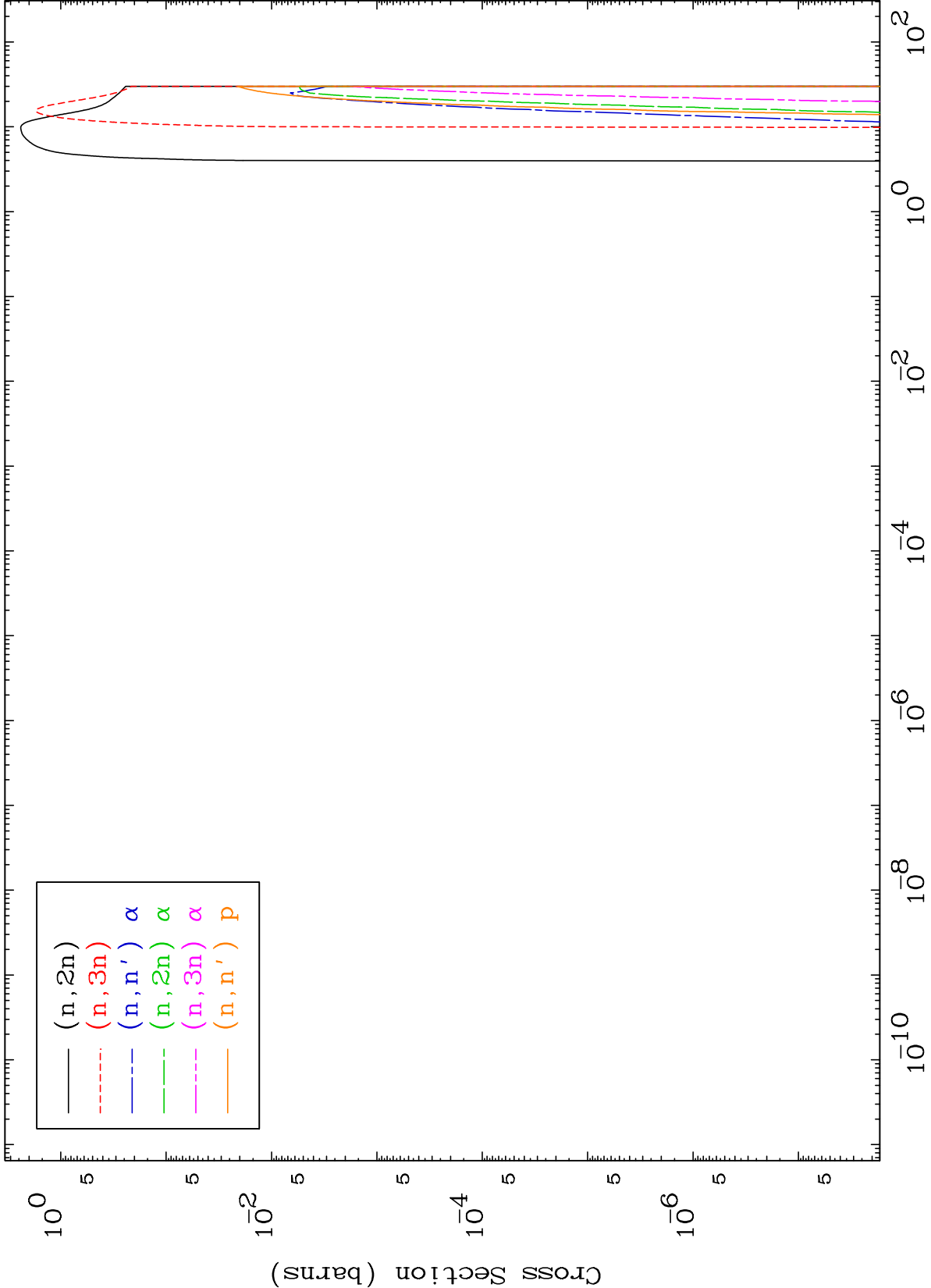
Press Mouse Button to Start

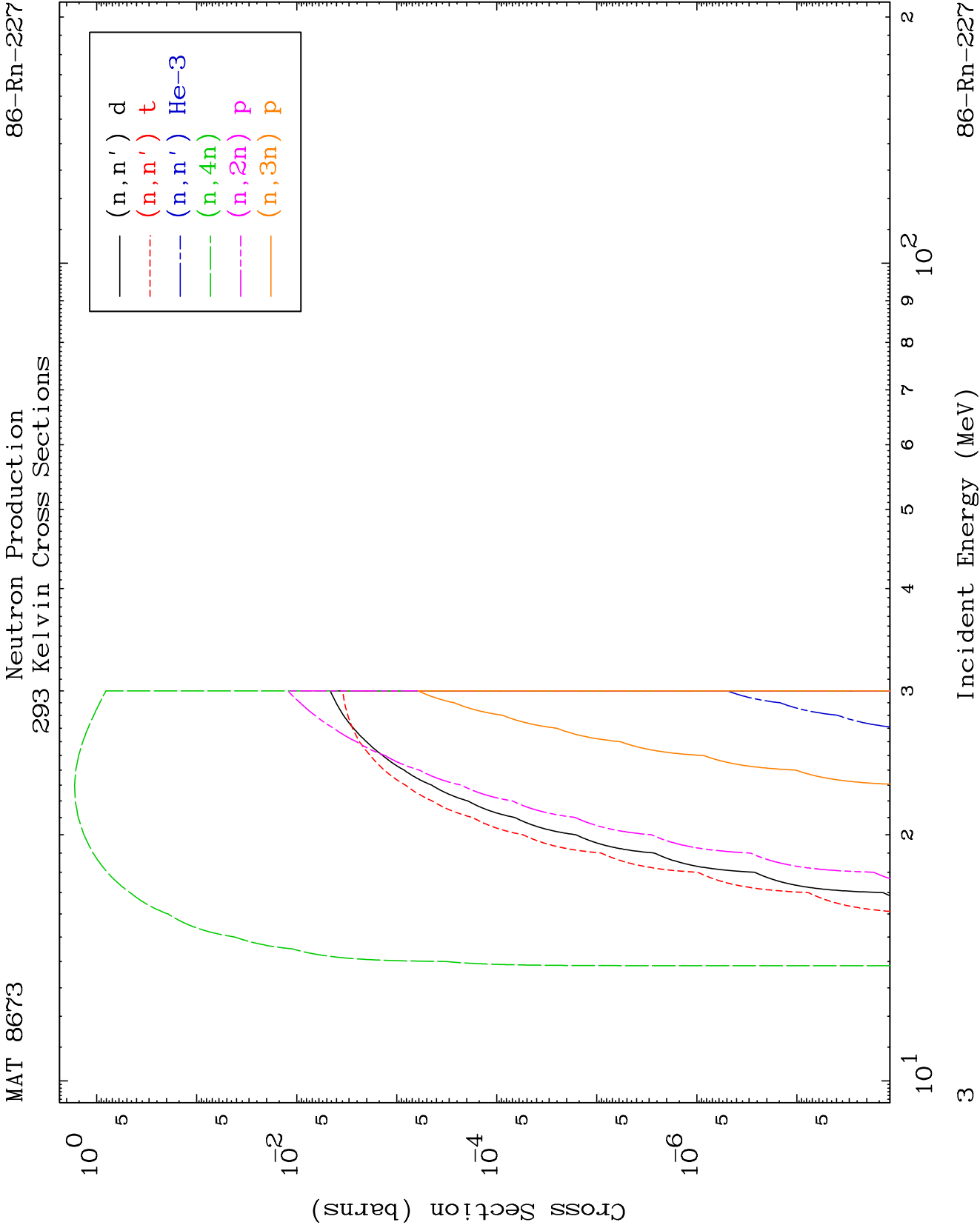


MAT 8673

Neutron Production
293 Kelvin Cross Sections

86-Rn-227

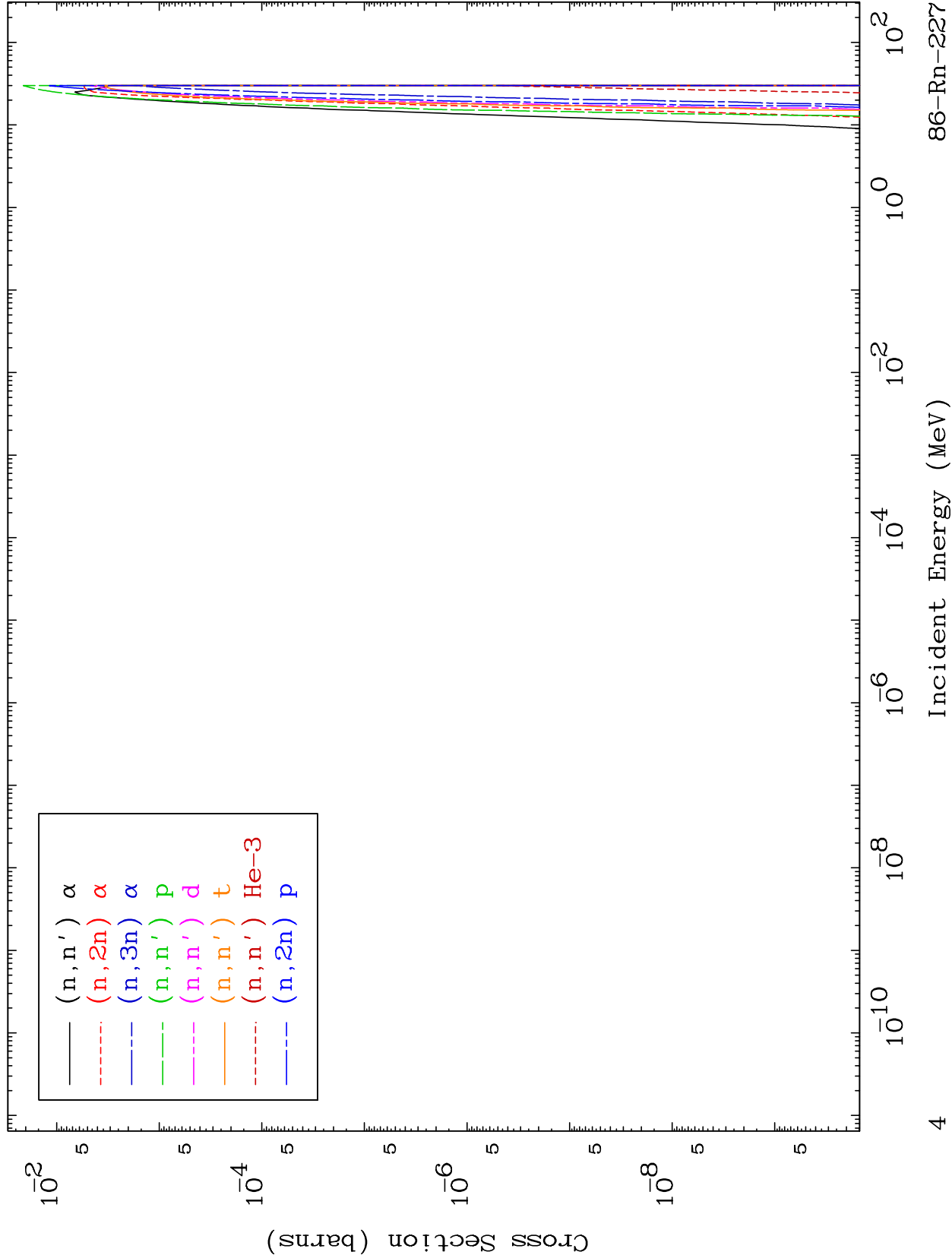


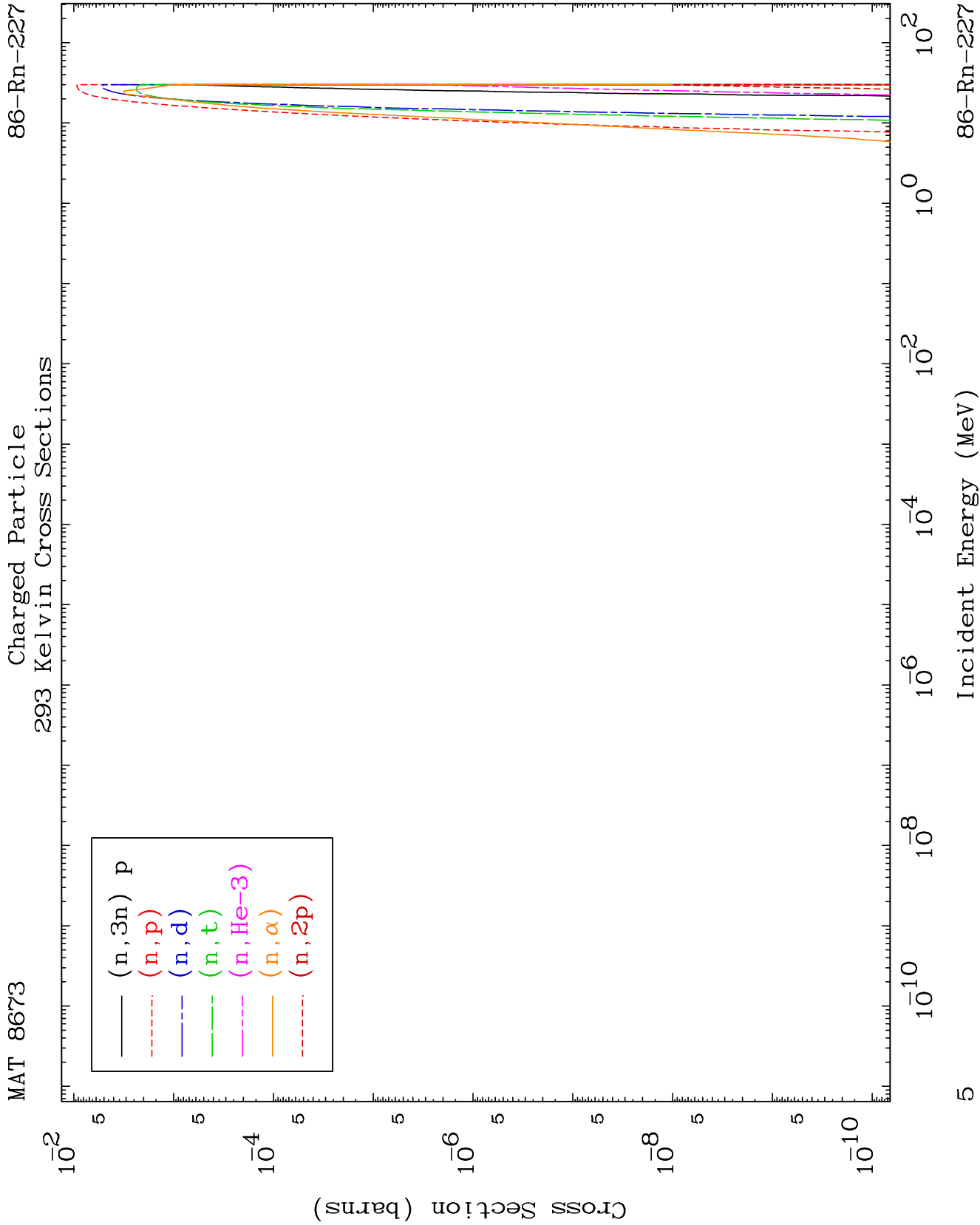


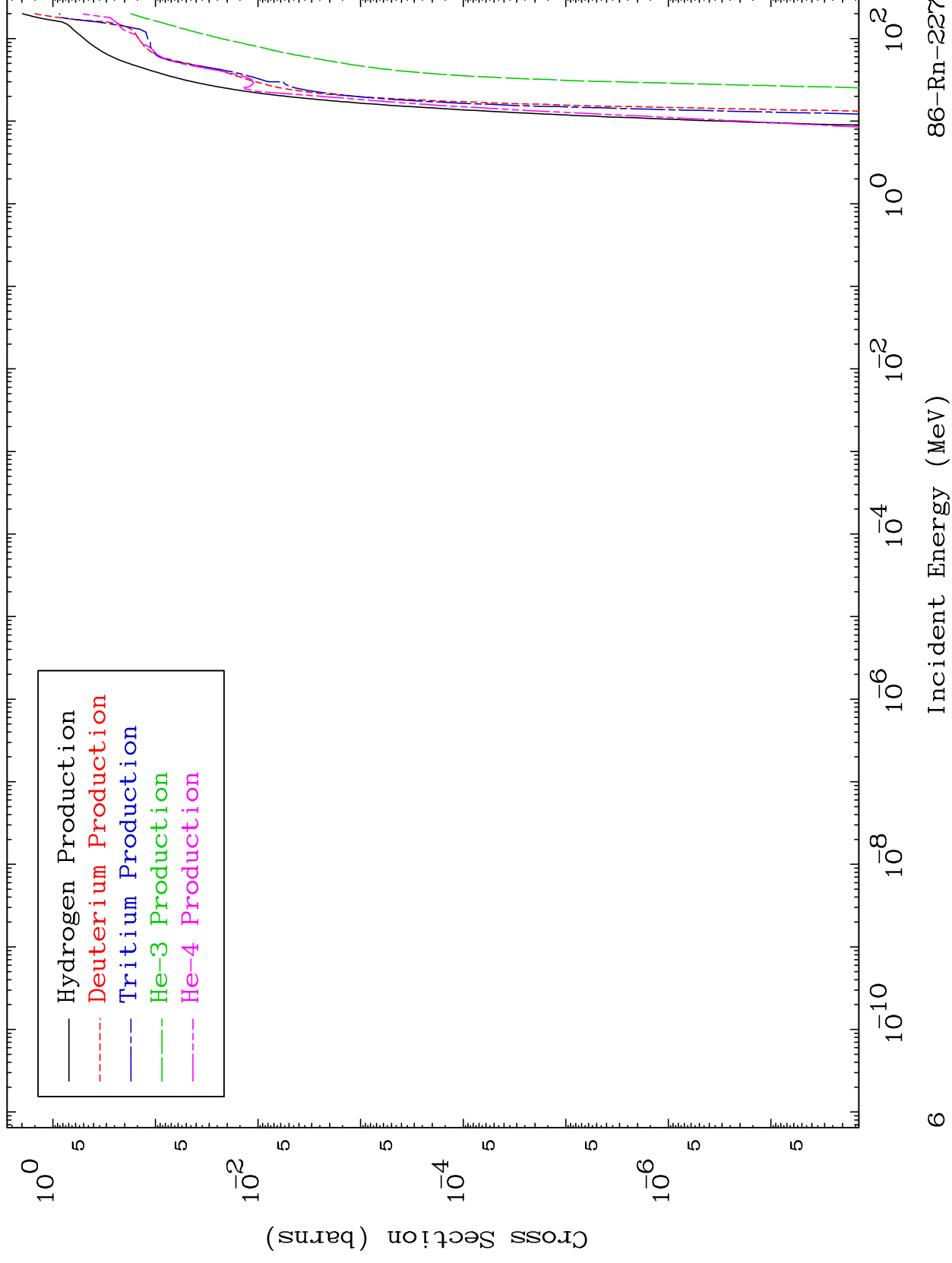
MAT 8673

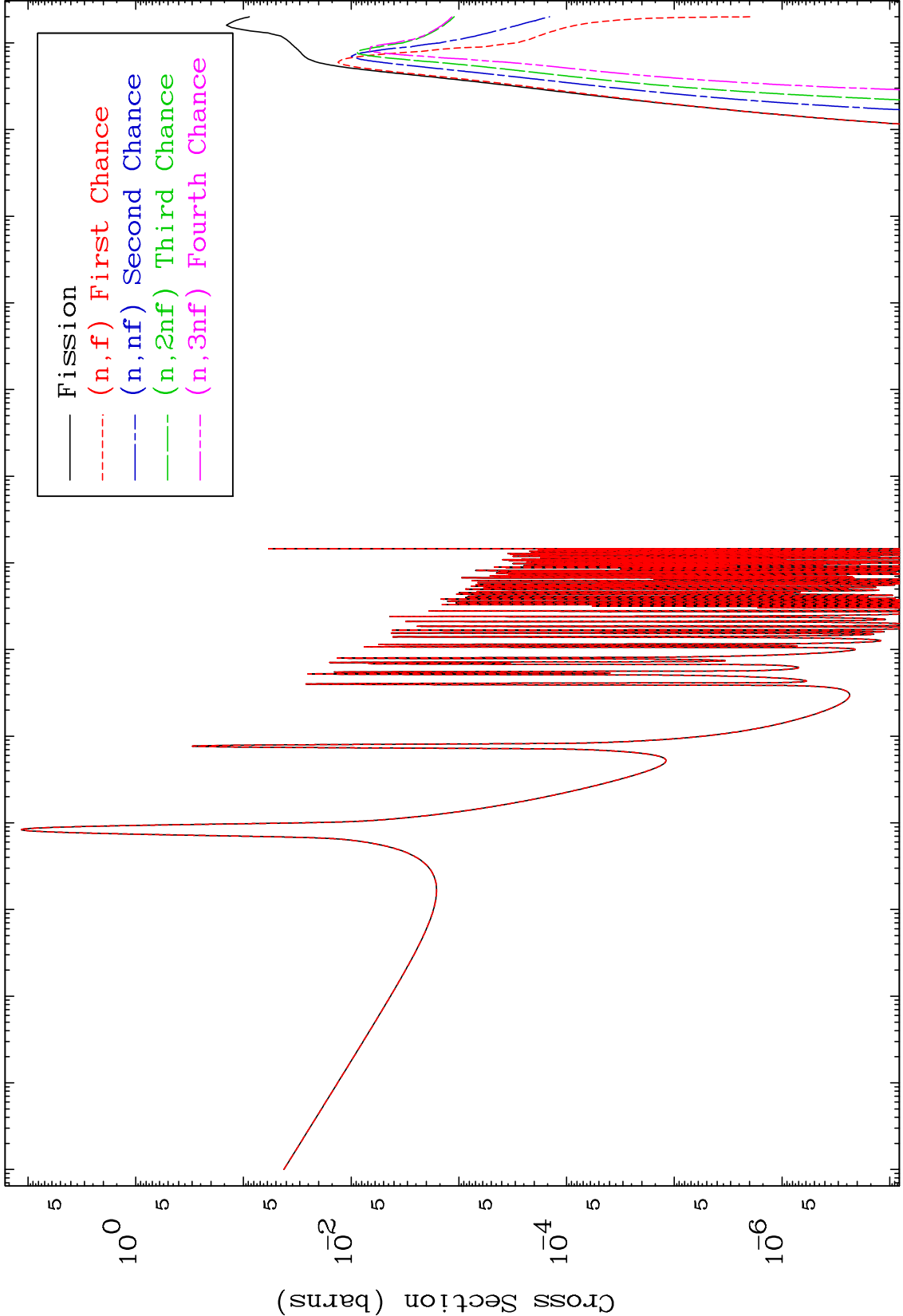
Charged Particle
293 Kelvin Cross Sections

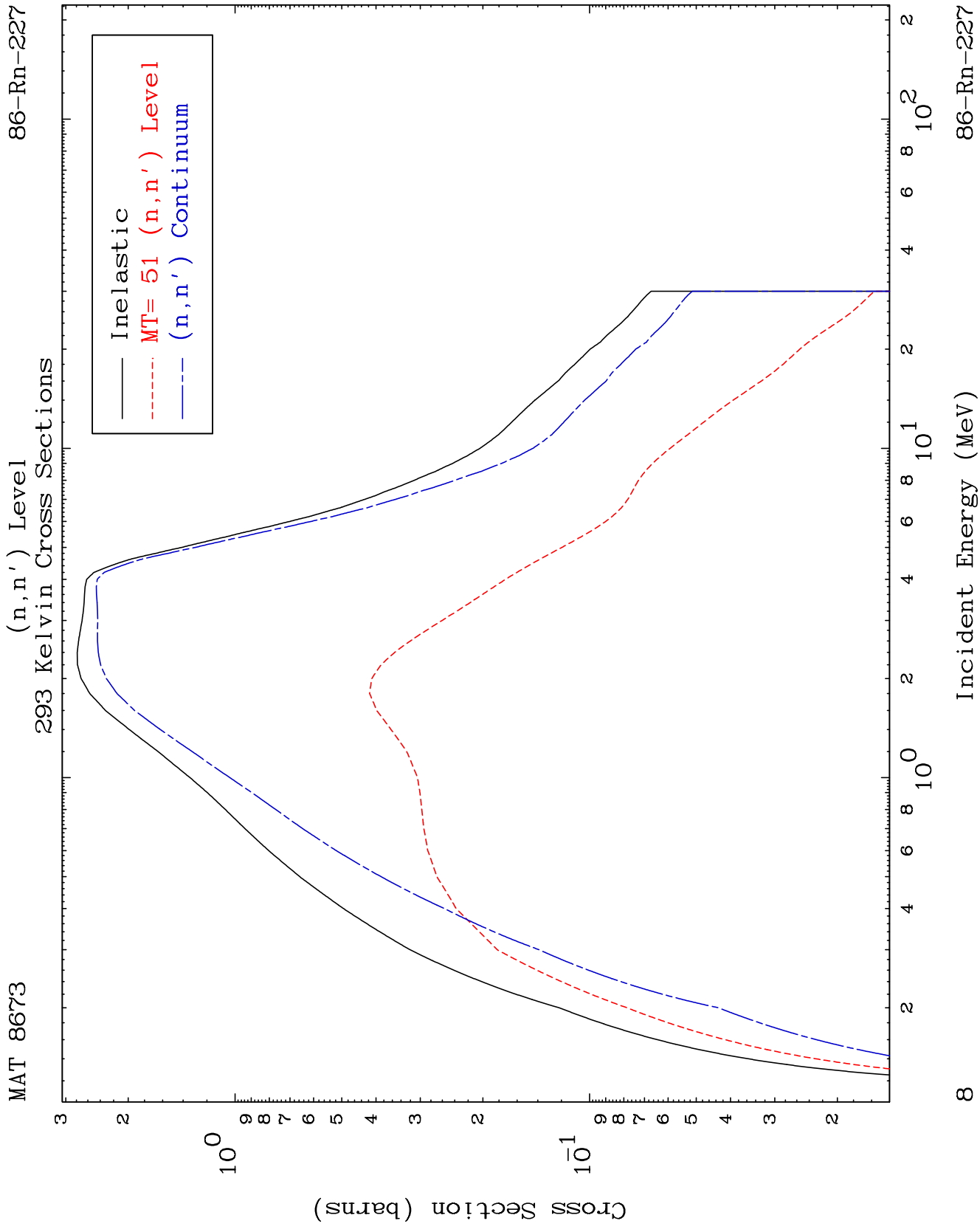
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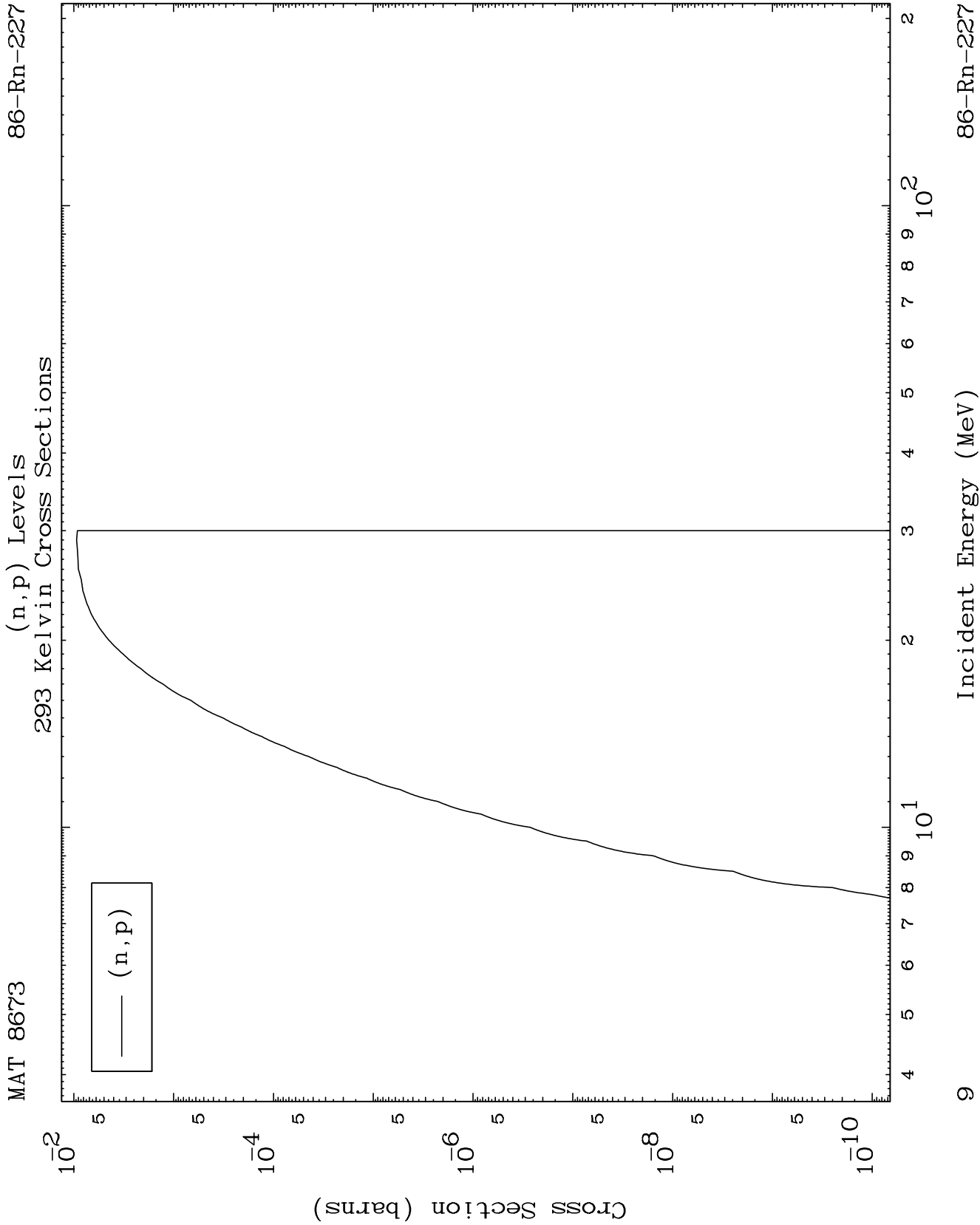


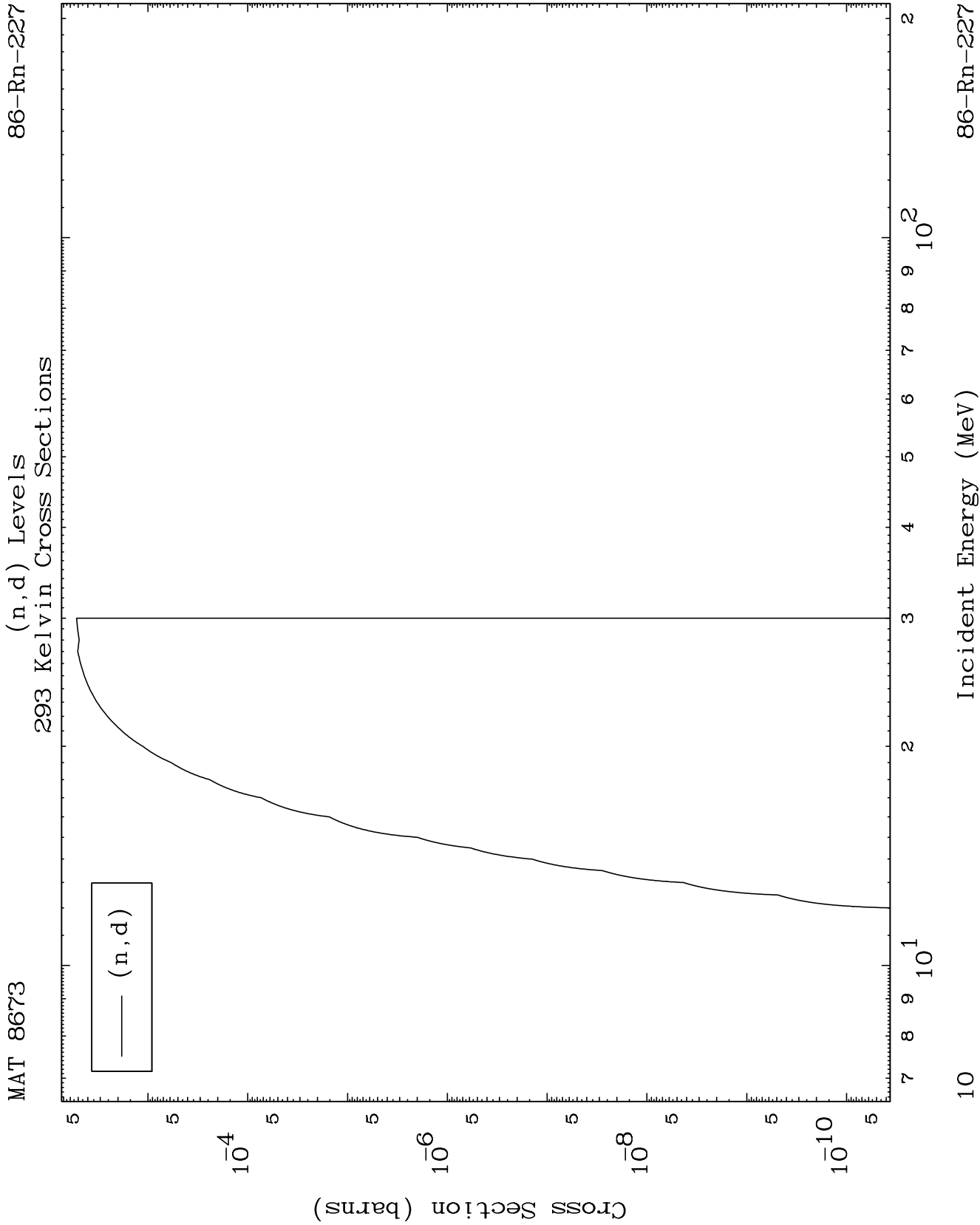








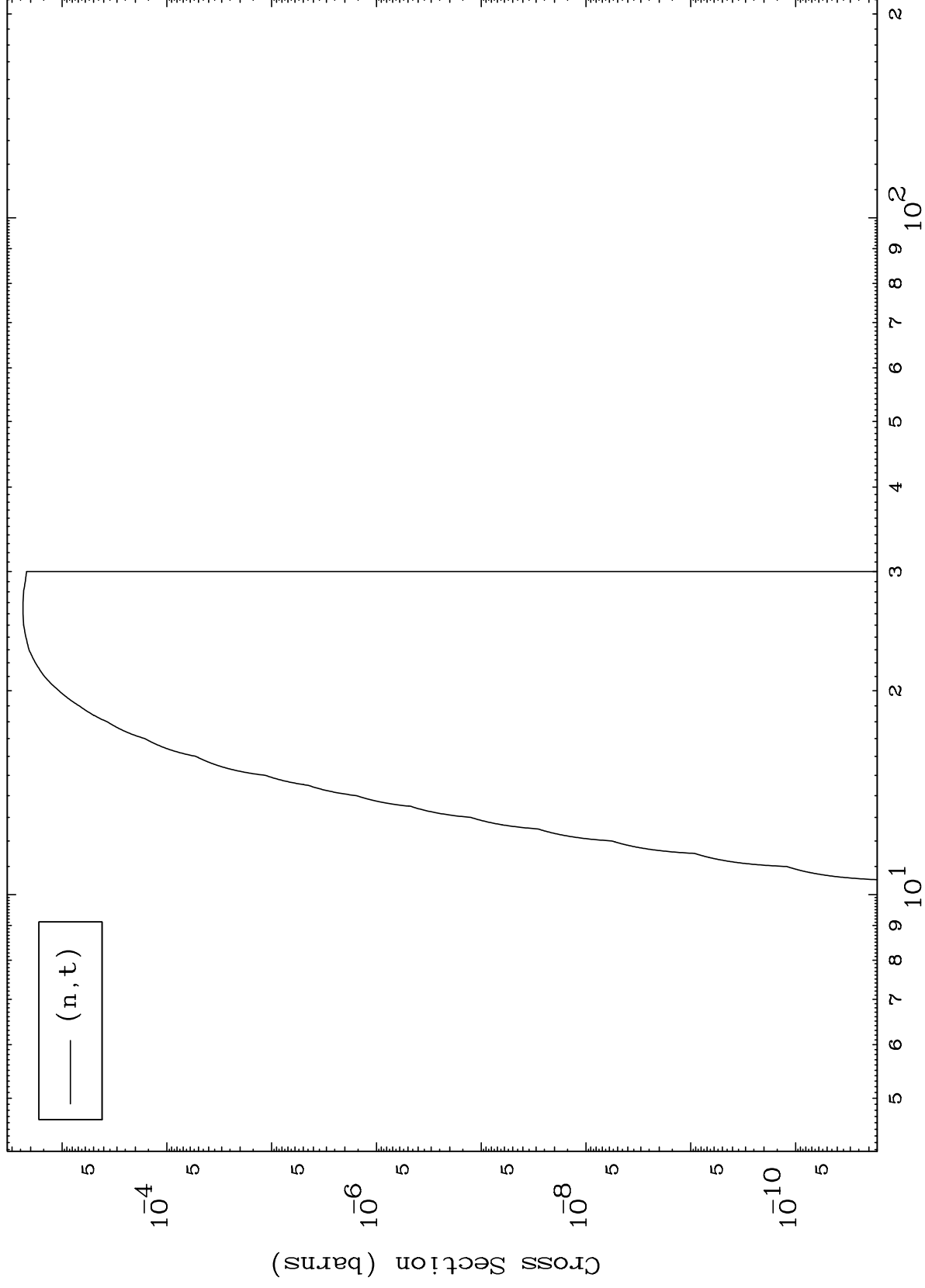




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(n,t) Levels
293 Kelvin Cross Sections



11

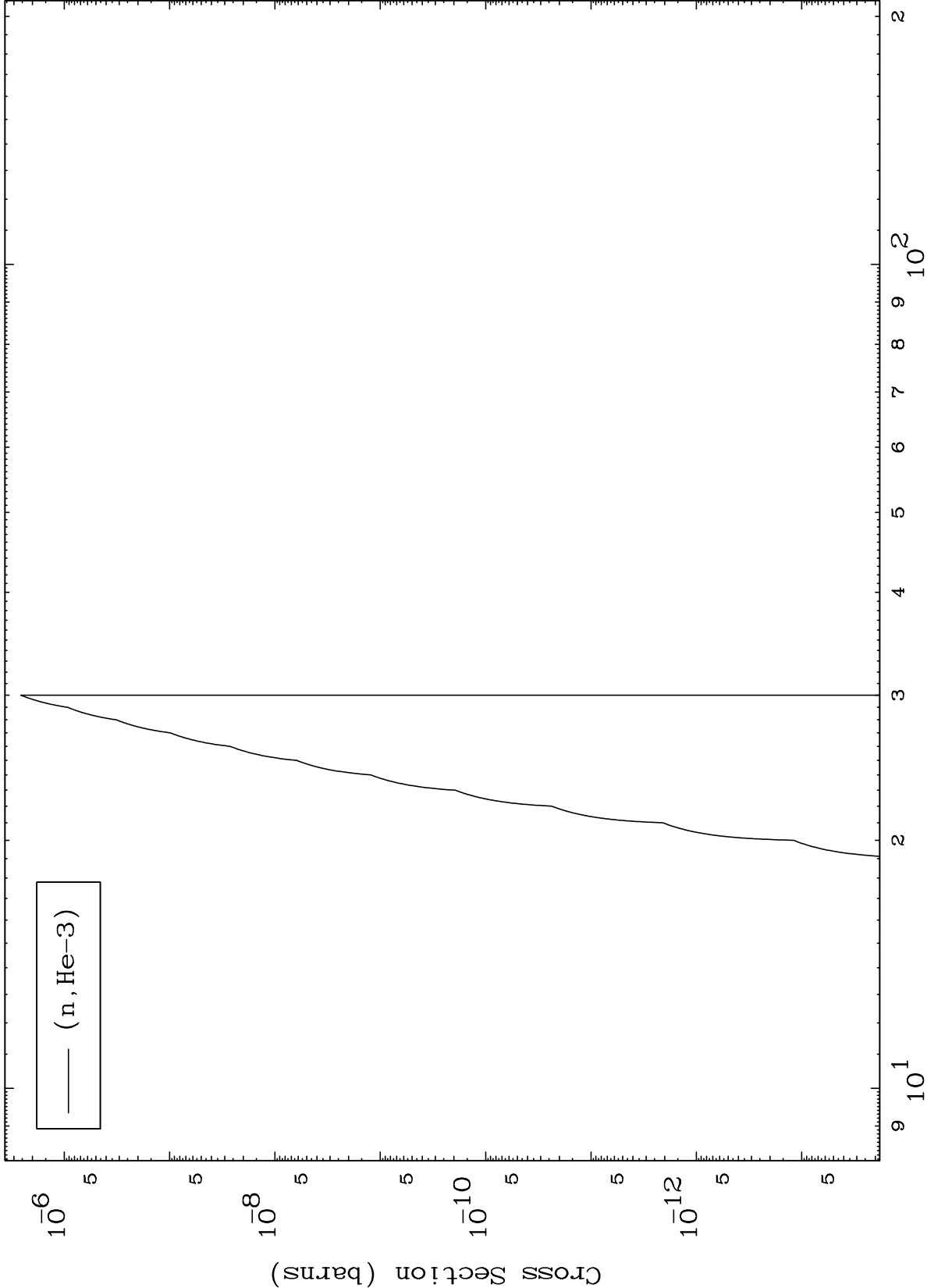
86-Rn-227

Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

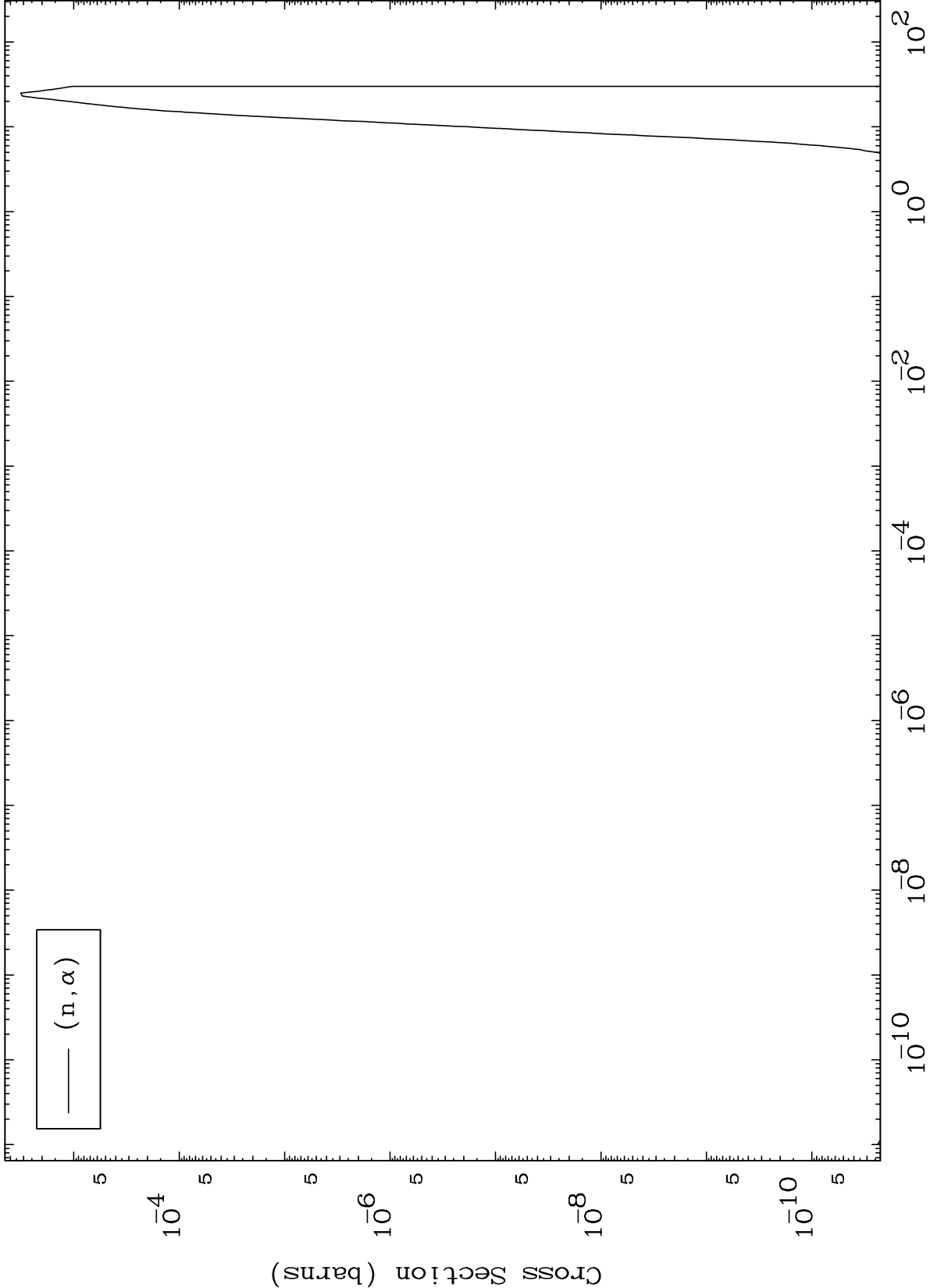
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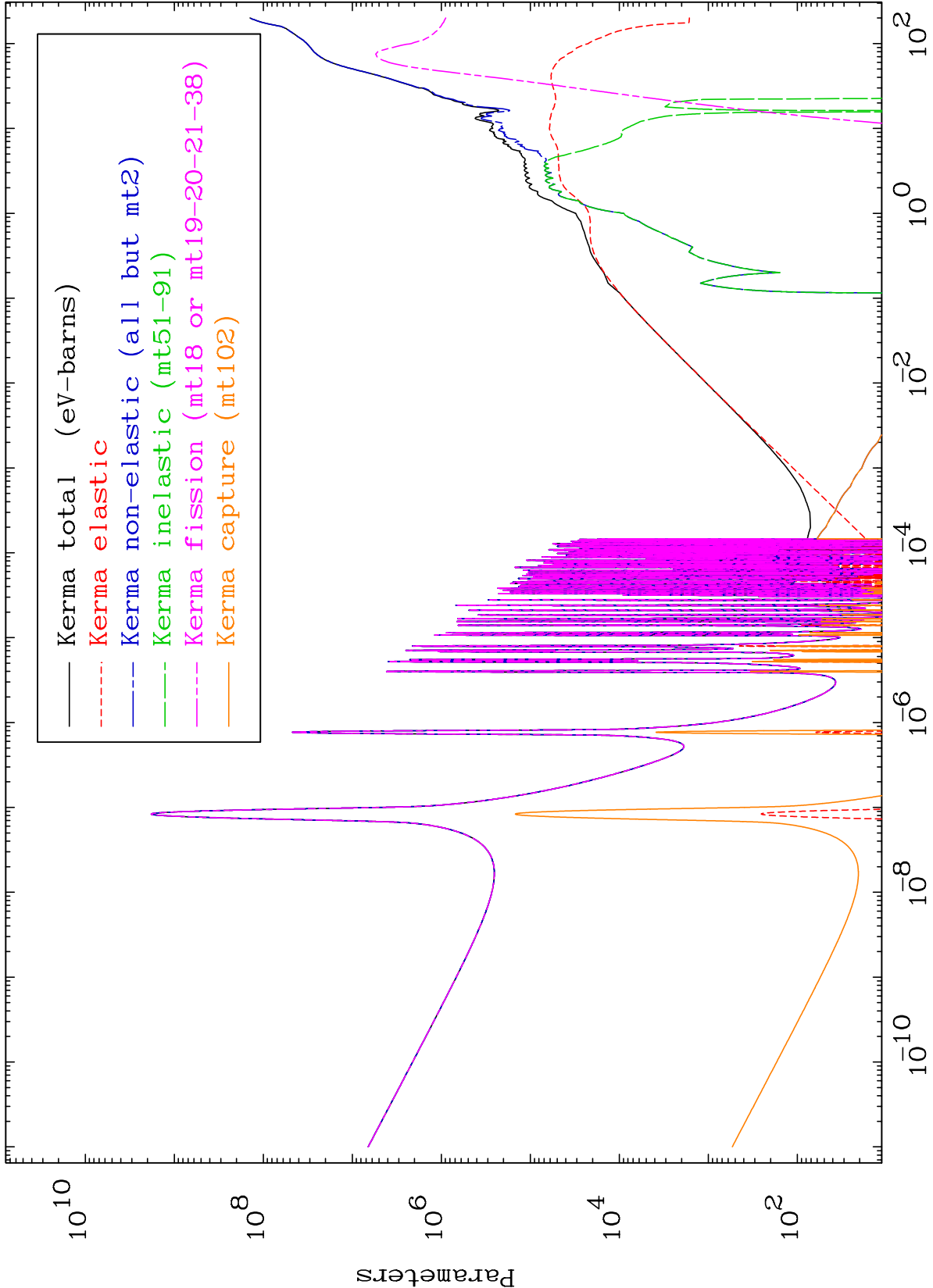


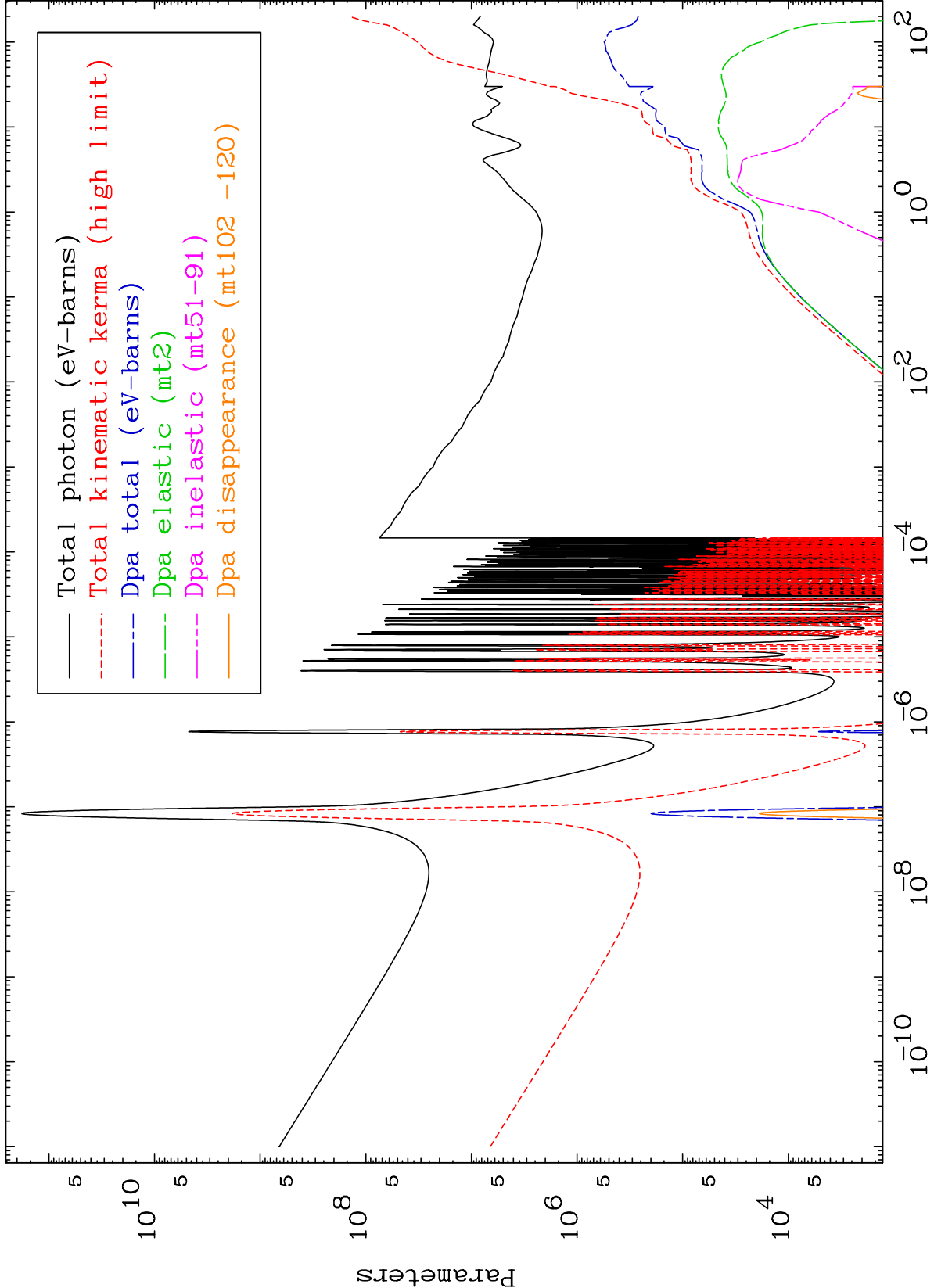
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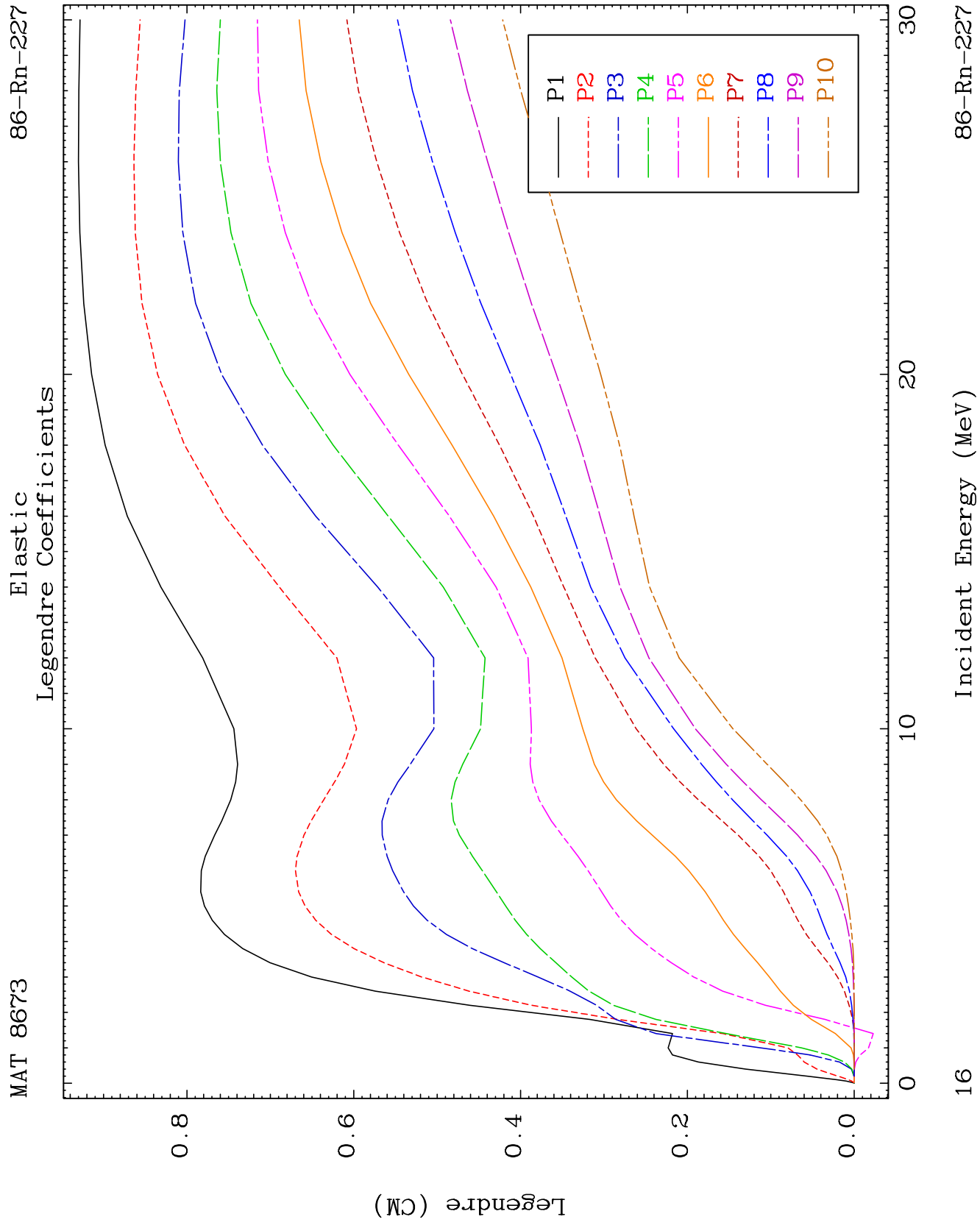
Incident Energy (MeV)

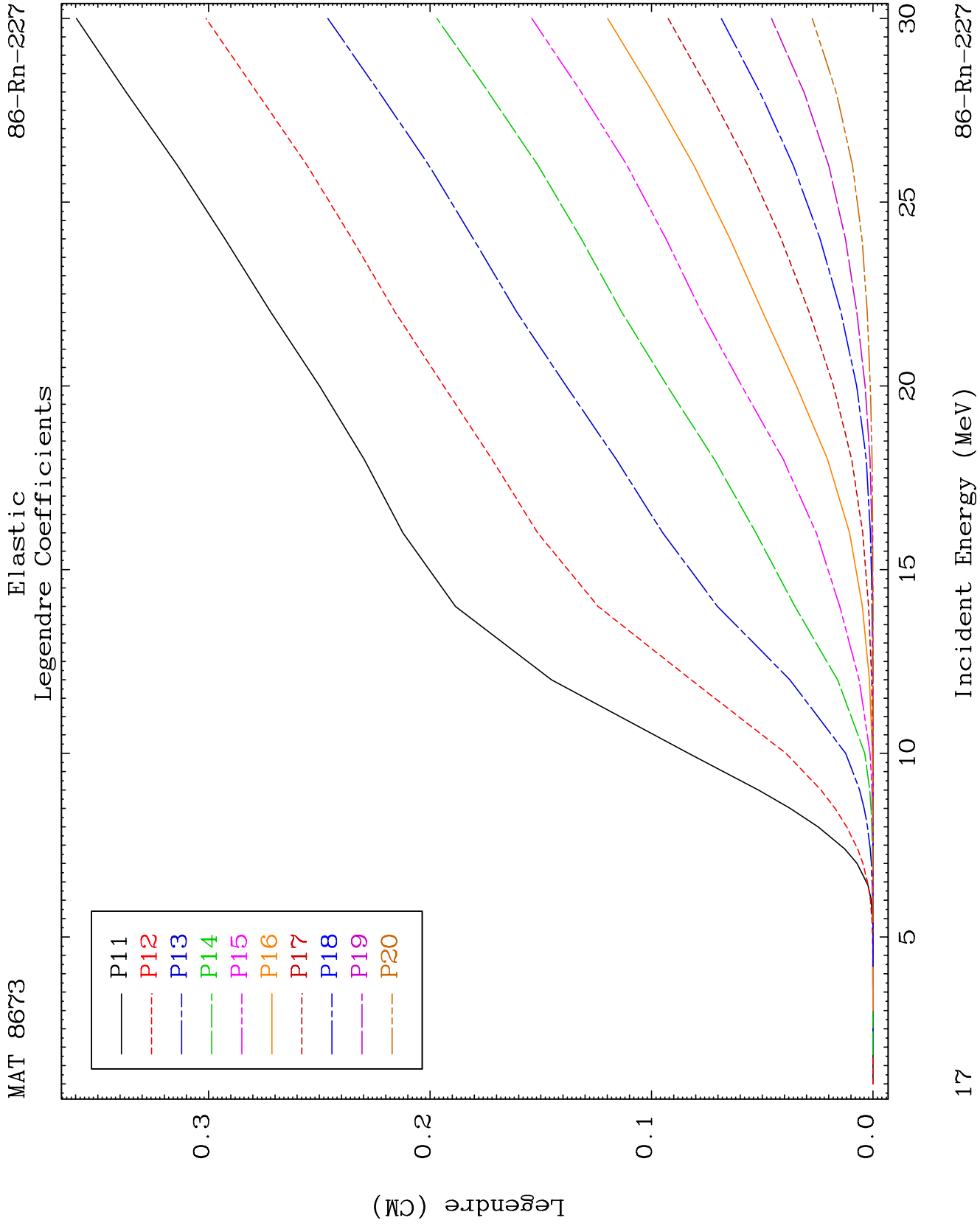
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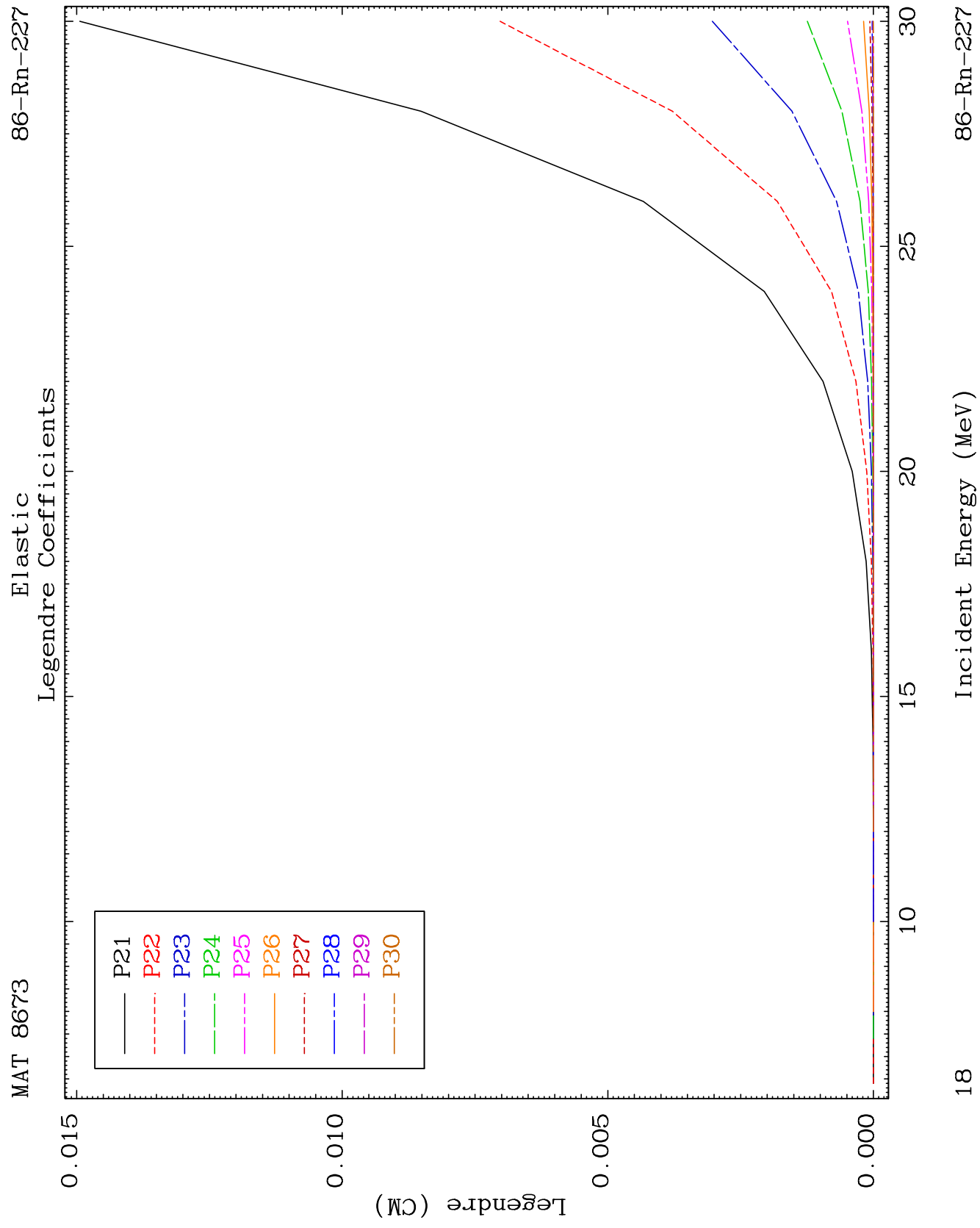


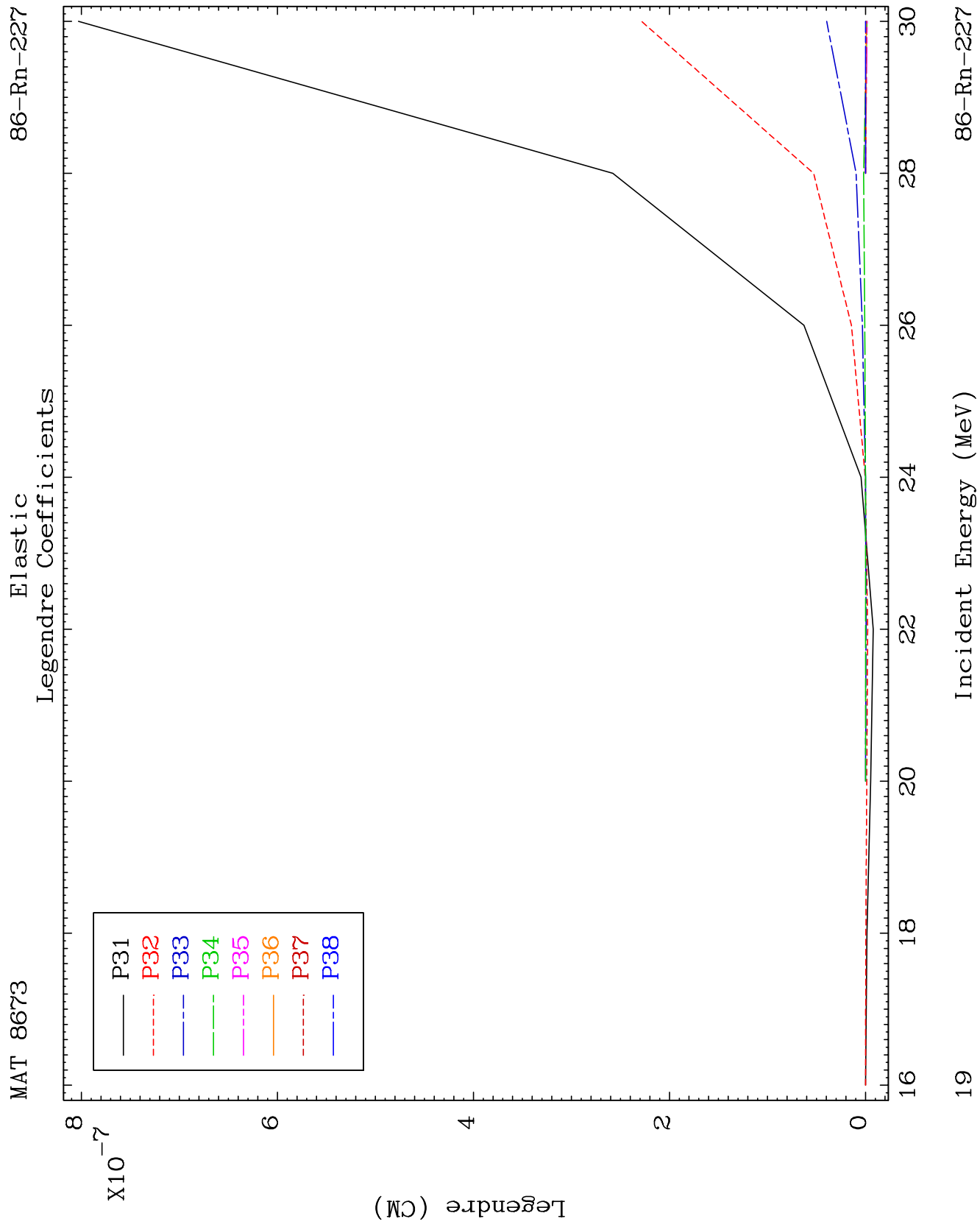








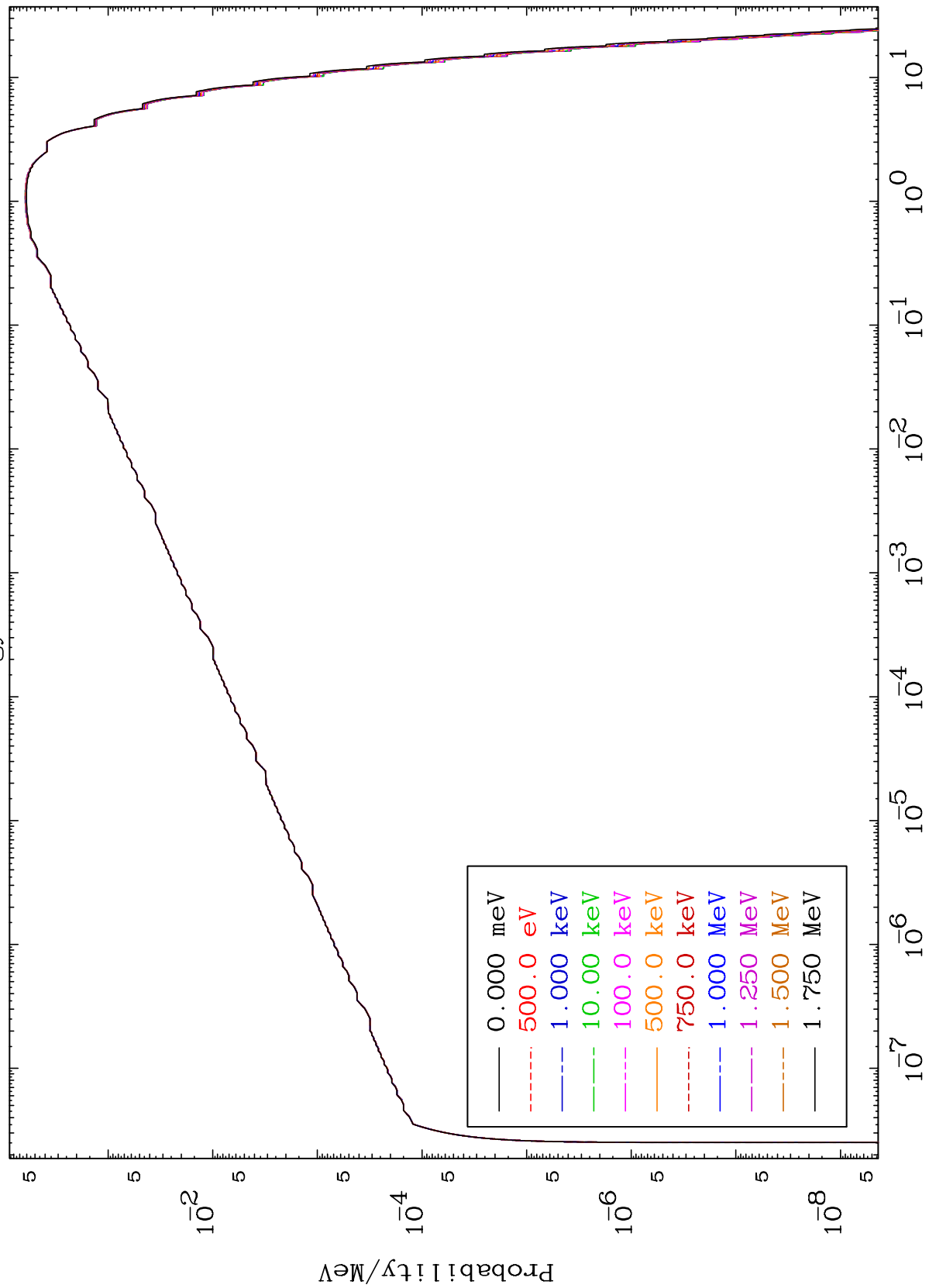




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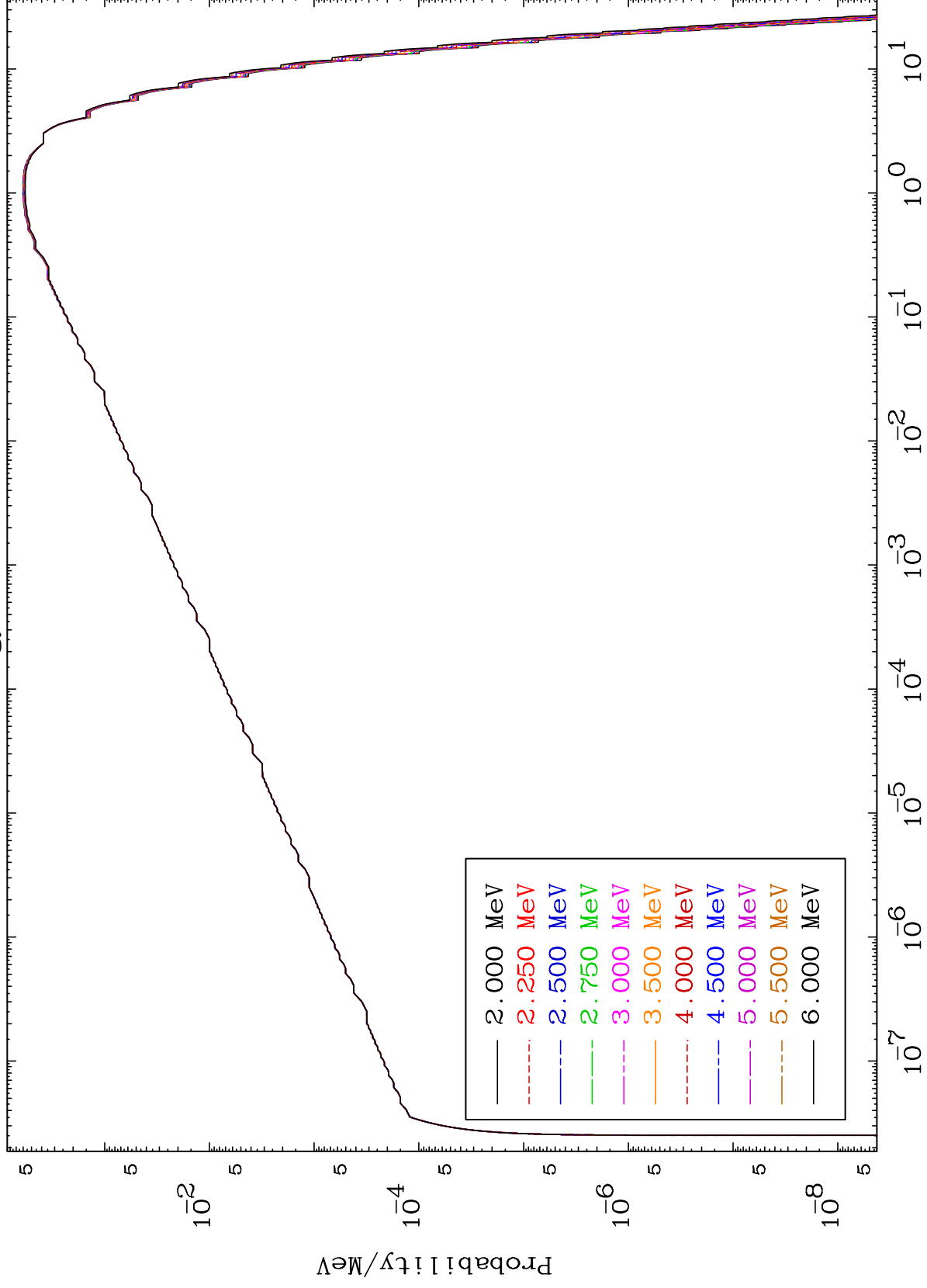
Fission
Energy Distribution

86-Rn-227



Fission
Energy Distribution

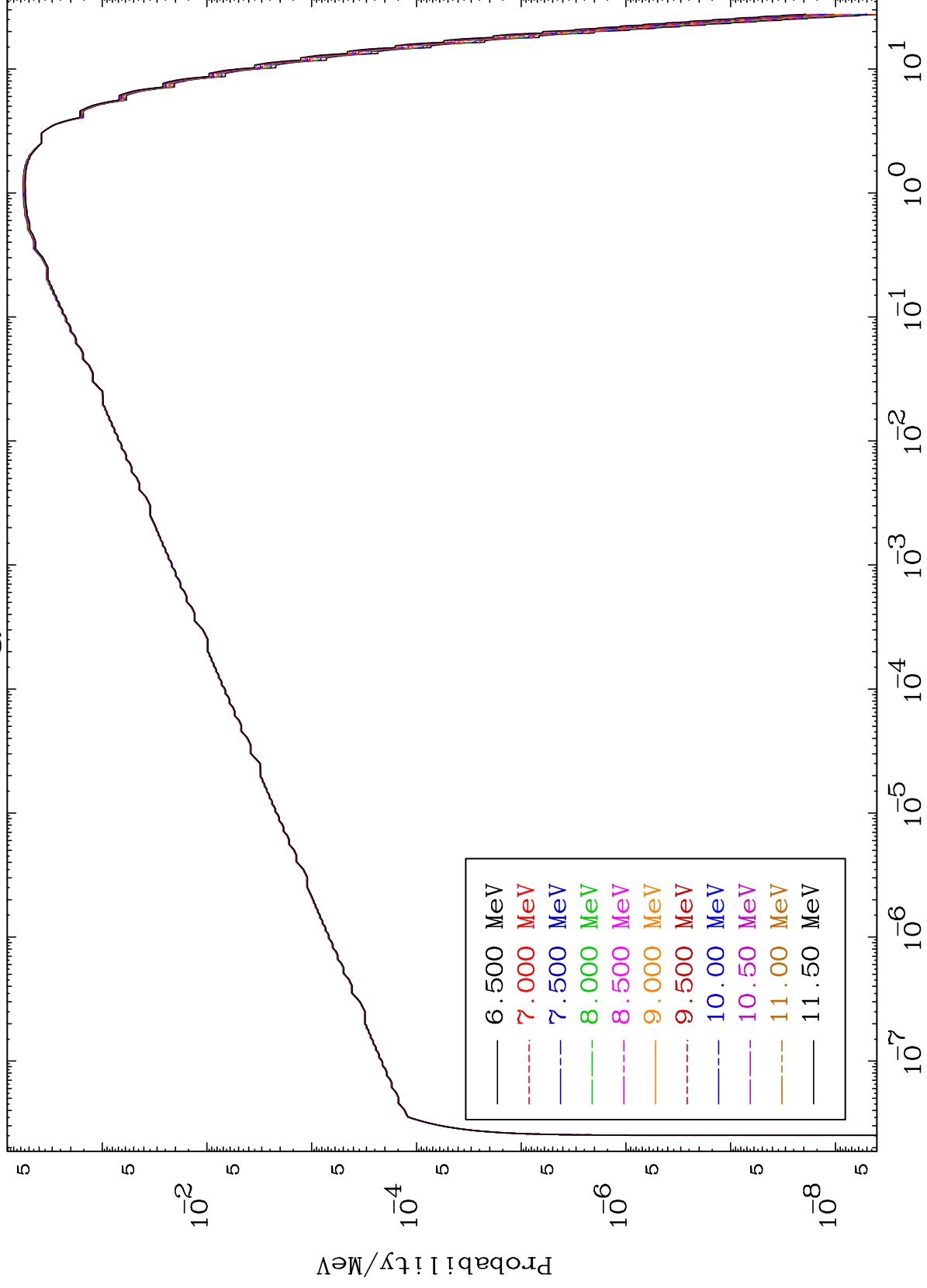
86-Rn-227



86-Rn-227

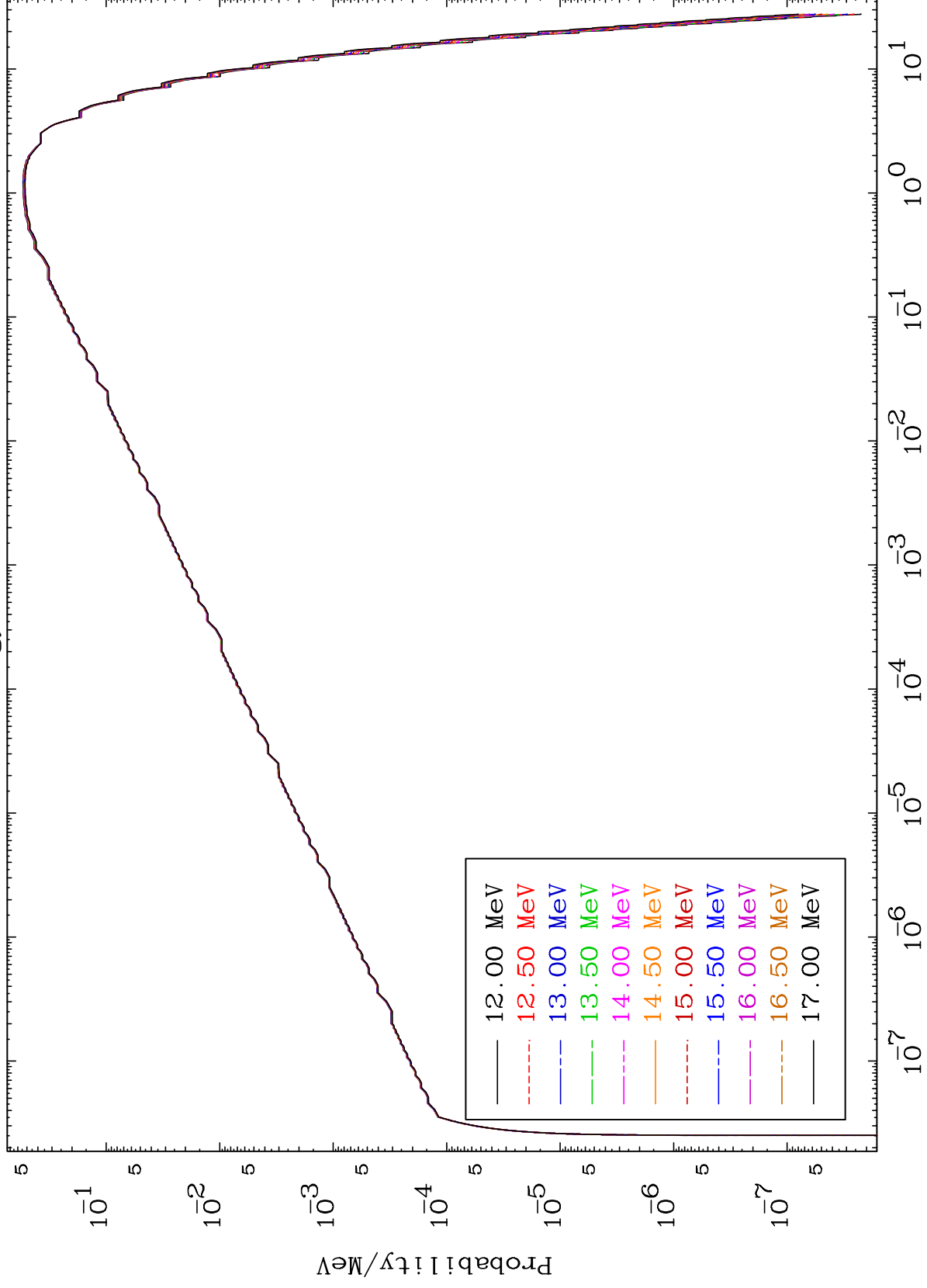
Fission
Energy Distribution

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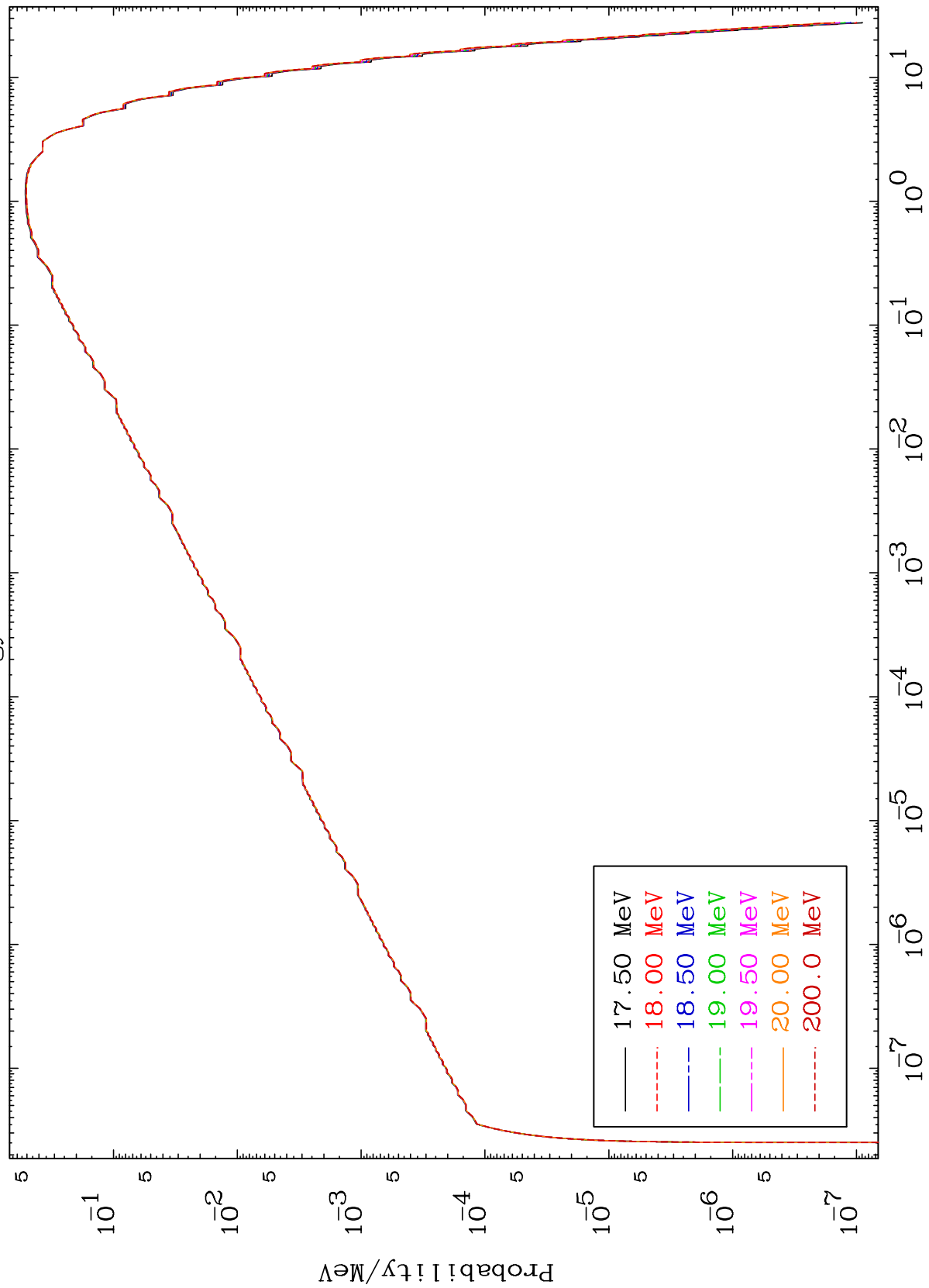
Fission
Energy Distribution

86-Rn-227



Fission Energy Distribution

86-Rn-227



86-Rn-227

Secondary Energy (MeV)