

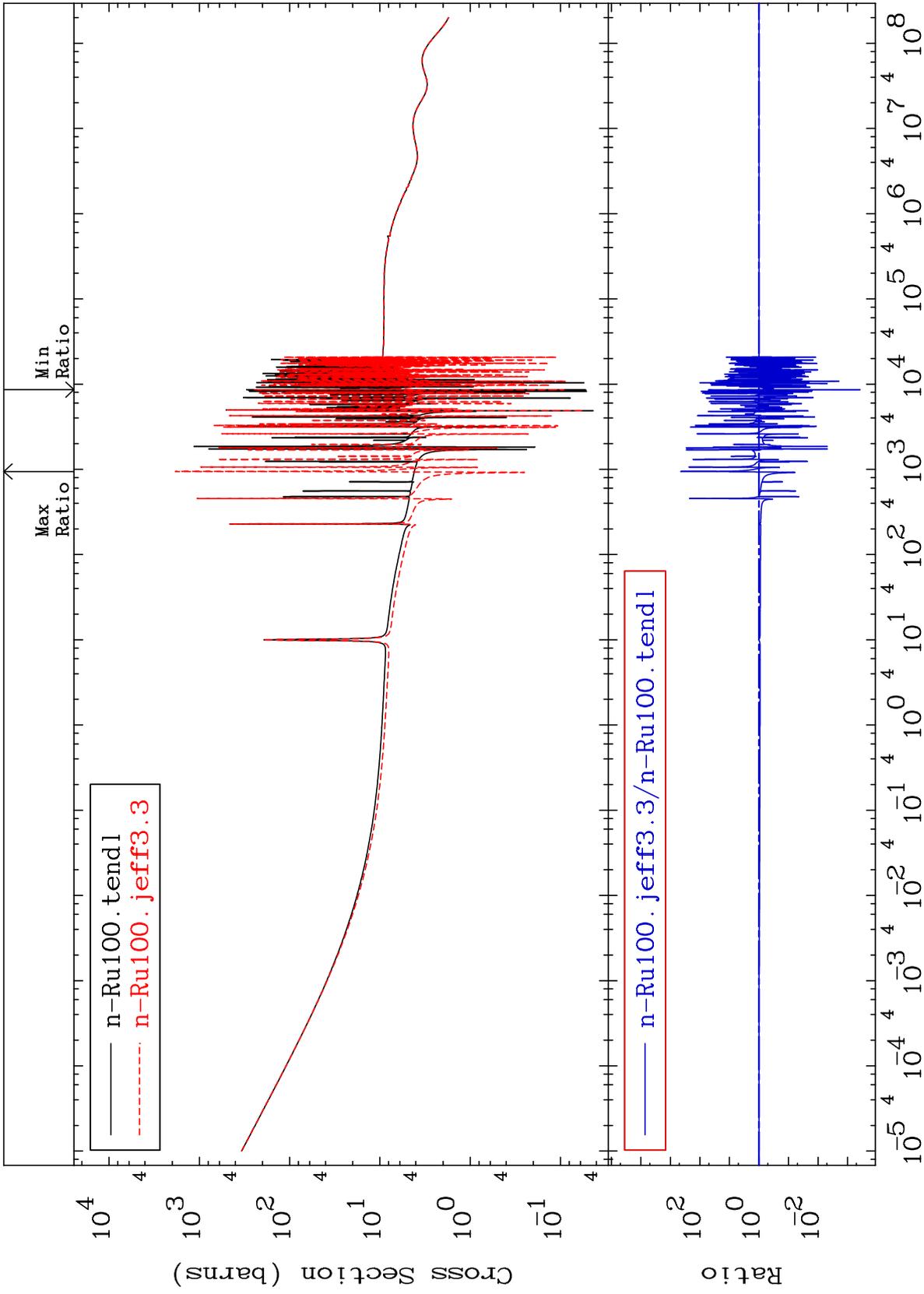
MAT 4437

Total

44-Ru-100

Cross Section

-99.96 To 9999. %



Incident Energy (eV)

44-Ru-100

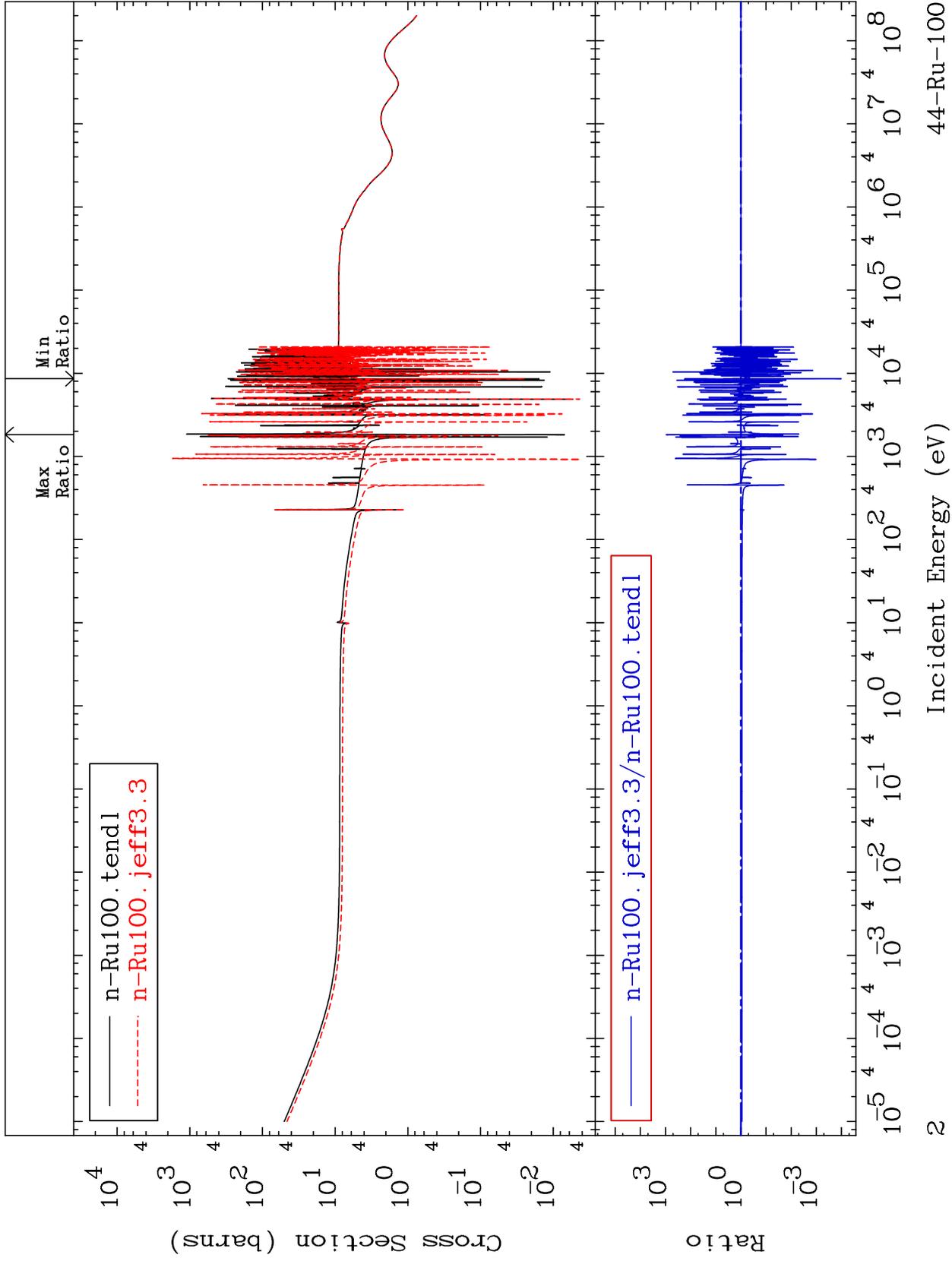
MAT 4437

Elastic

Cross Section

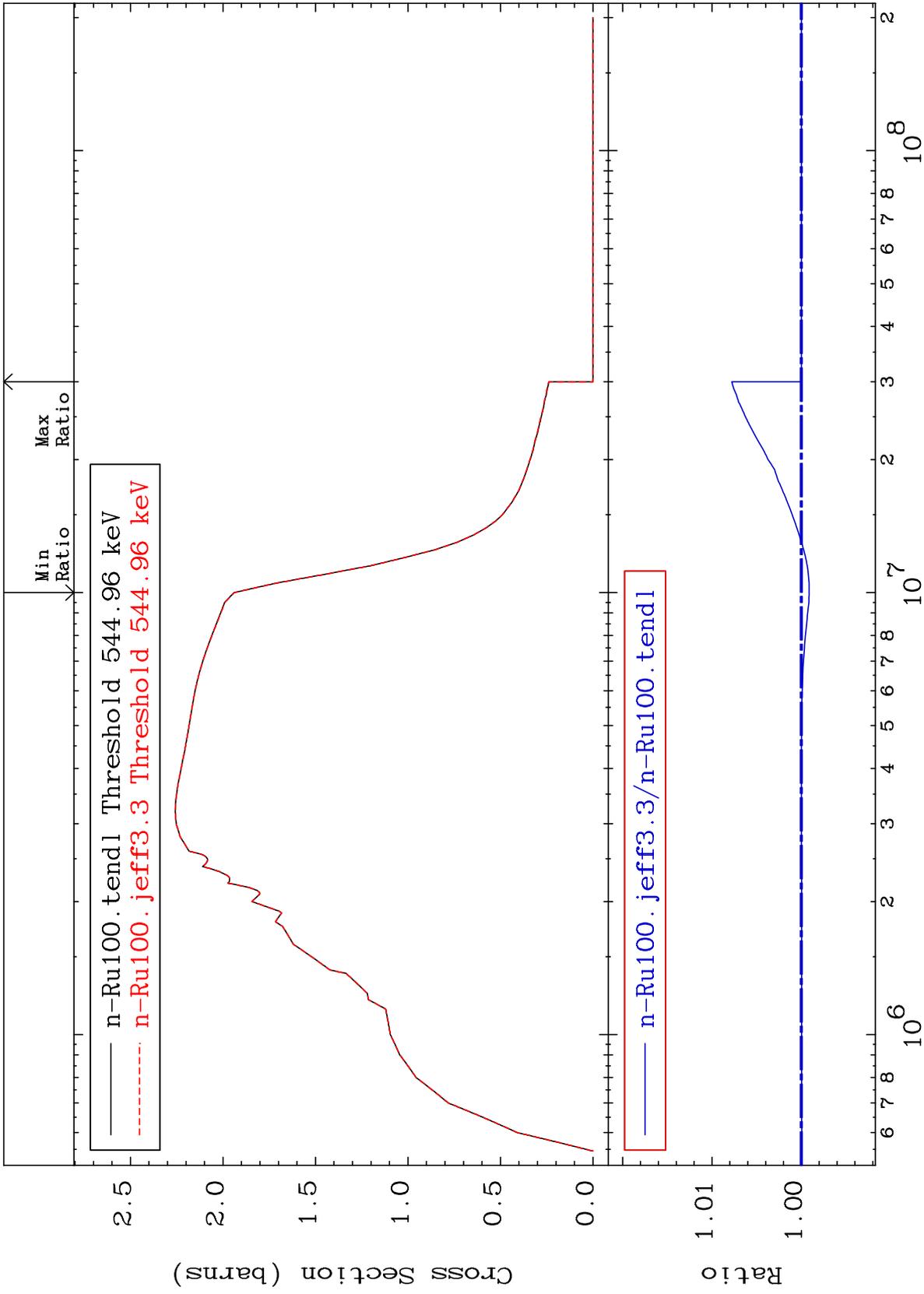
44-Ru-100

-99.99 To 9999. %



MAT 4437

Inelastic  
Cross Section  
44-Ru-100  
-0.087 To 0.779 %



3

Incident Energy (eV)

44-Ru-100

MAT 4437

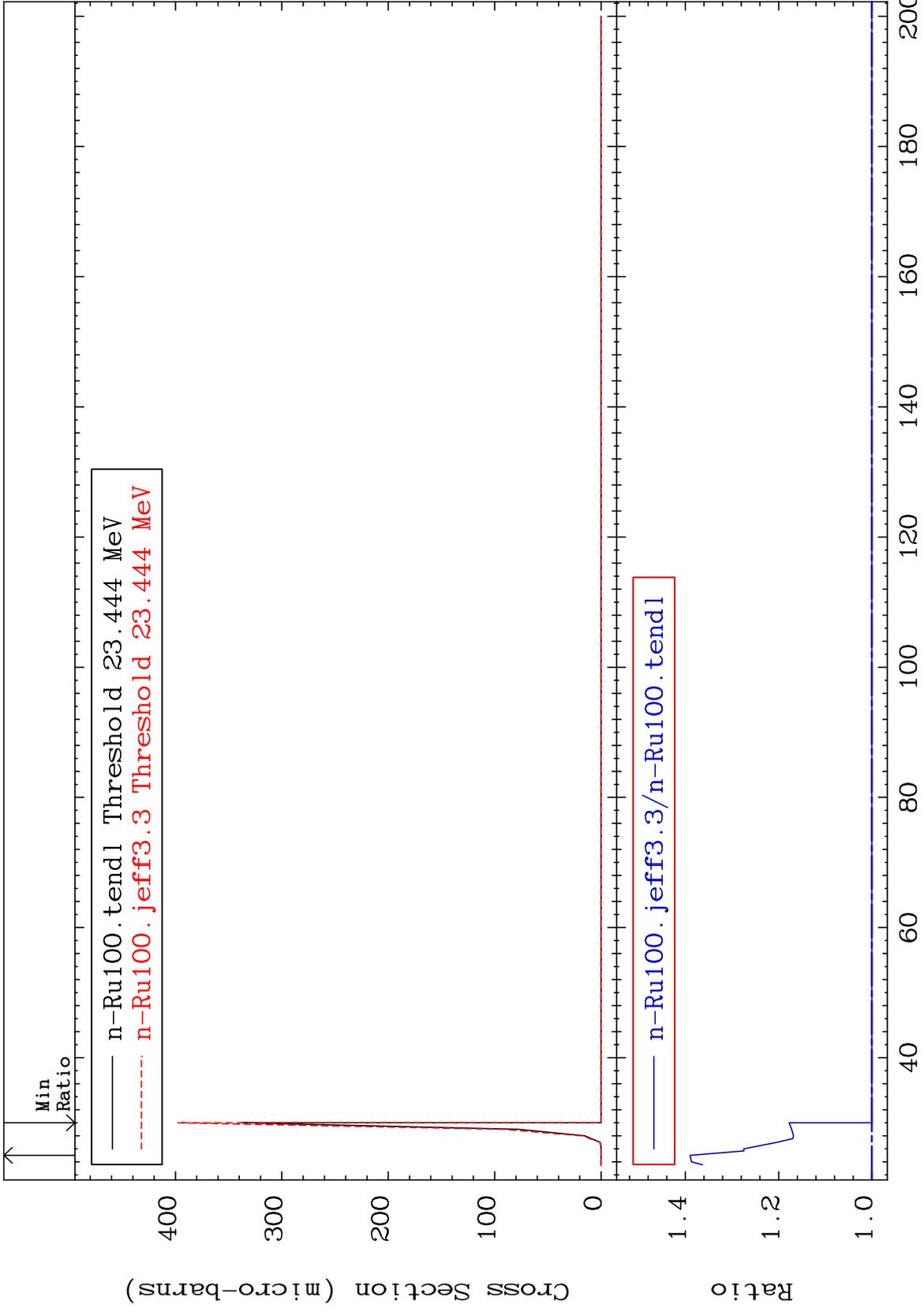
(n,2n) d

44-Ru-100

Cross Section

0.000

To 38.99 %



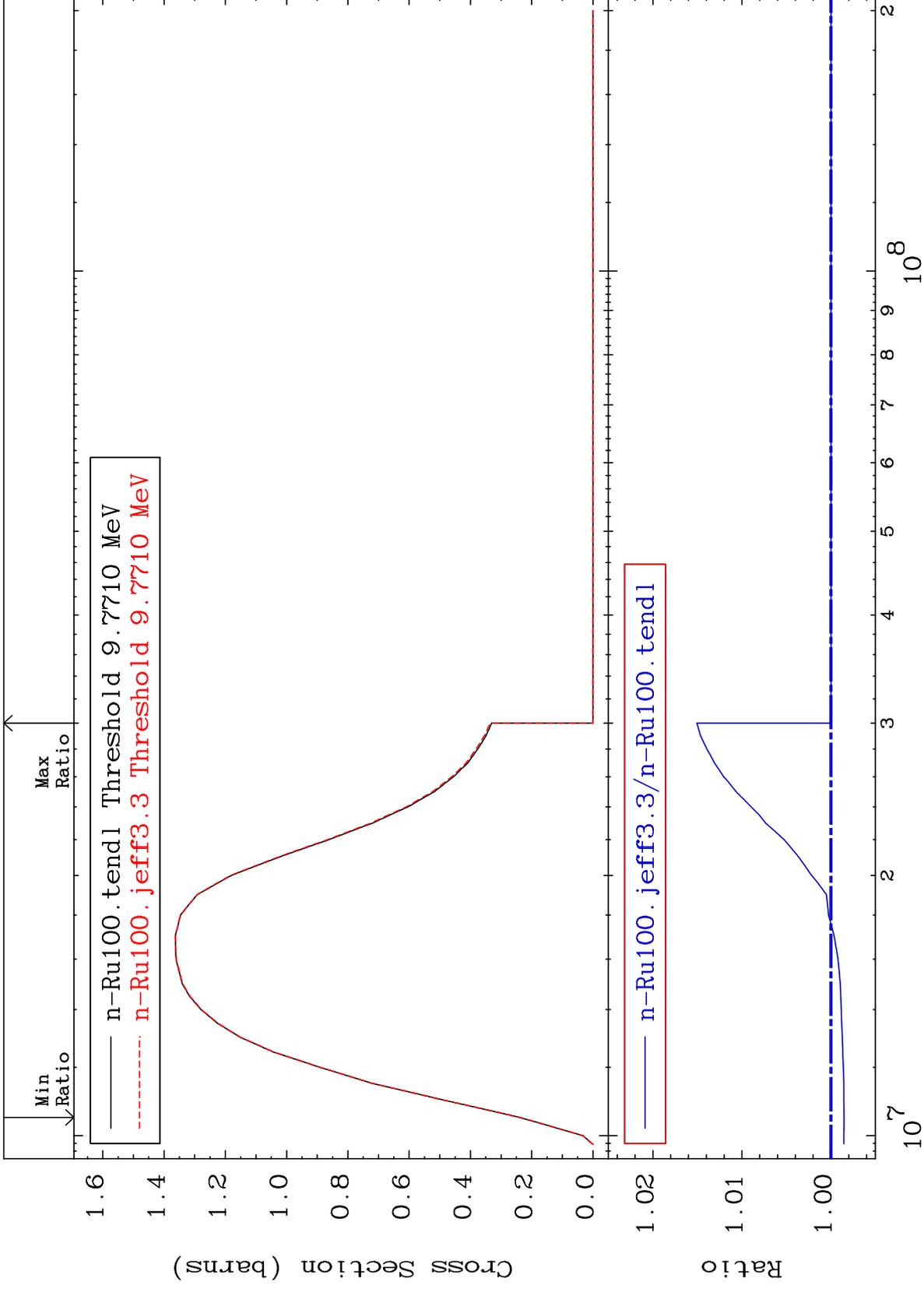
MAT 4437

(n,2n)

44-Ru-100

Cross Section

-0.146 To 1.506 %



MAT 4437

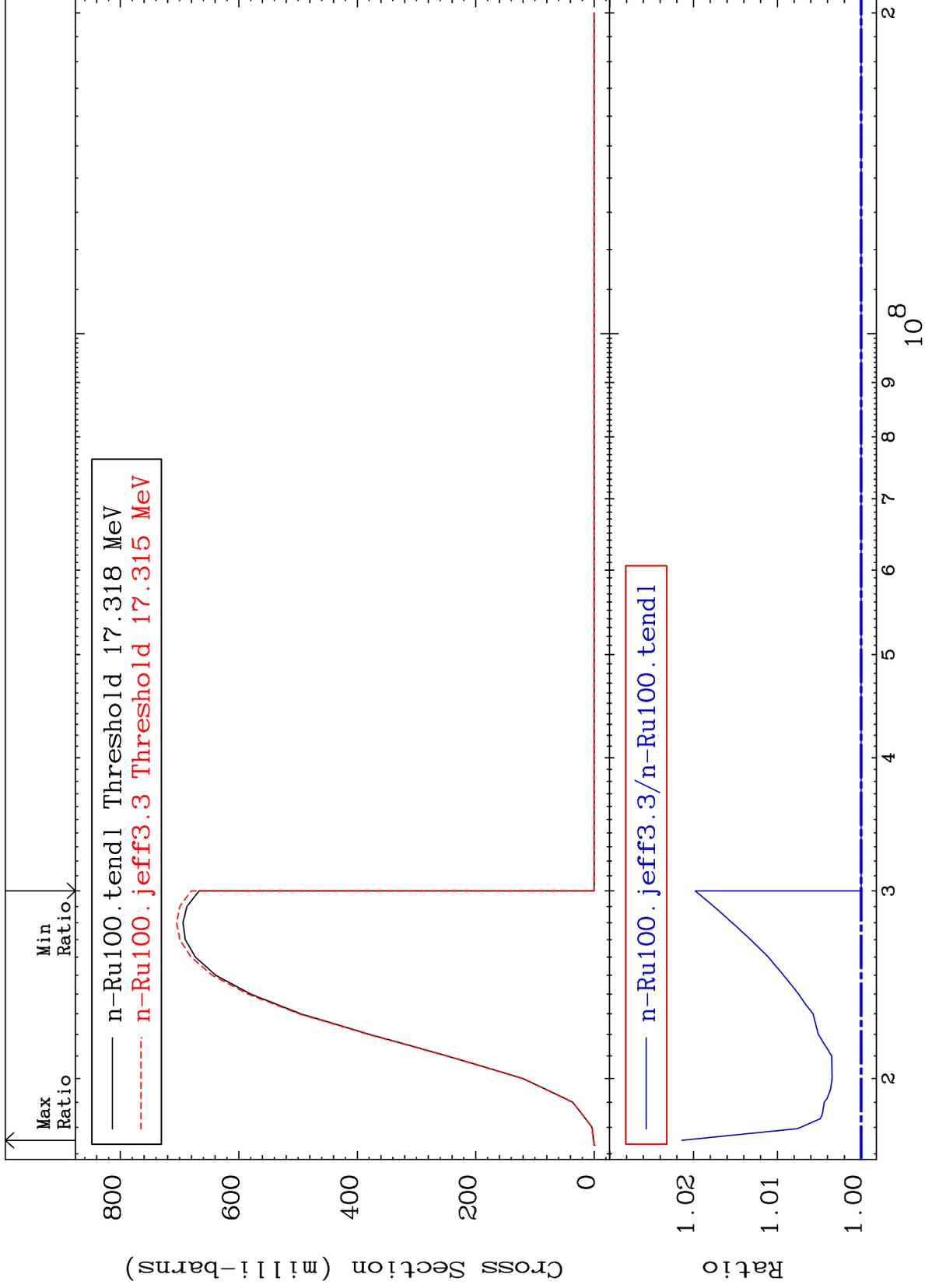
(n,3n)

44-Ru-100

Cross Section

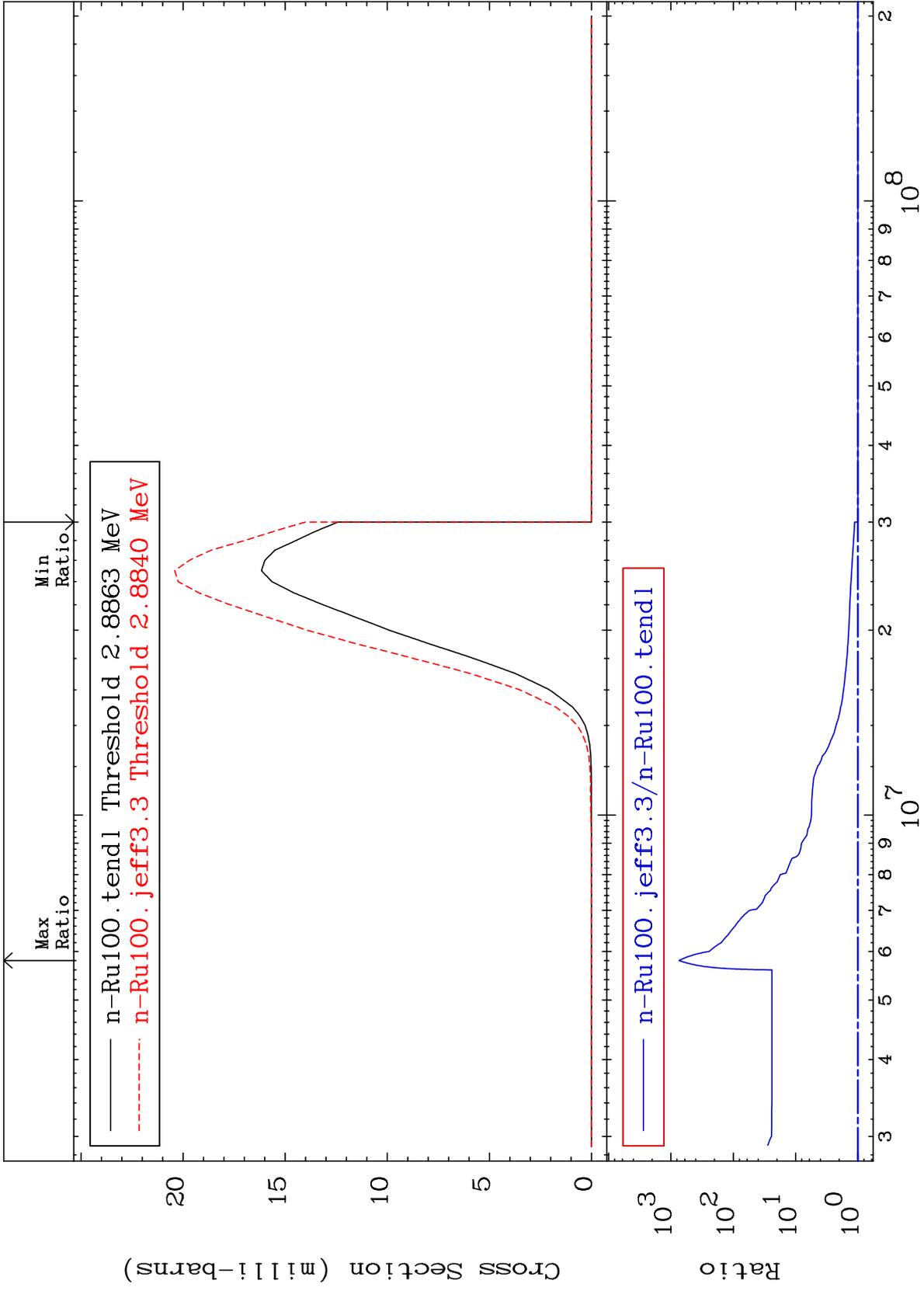
0.000

To 2.139 %



MAT 4437

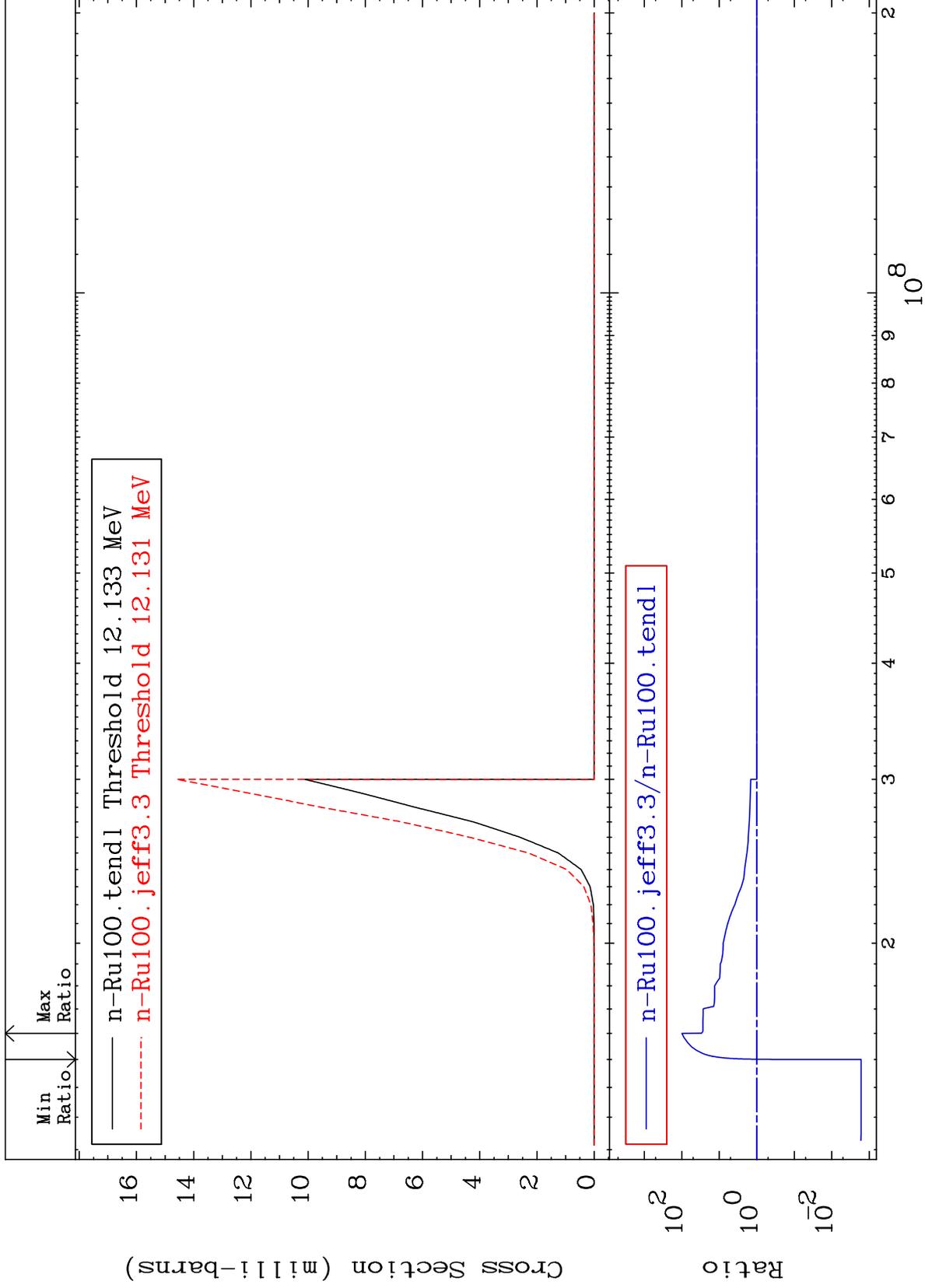
(n, n')  $\alpha$   
Cross Section  
0.000 To 9999. %  
44-Ru-100



MAT 4437

(n,2n)  $\alpha$   
Cross Section

44-Ru-100  
-99.84 To 9811. %

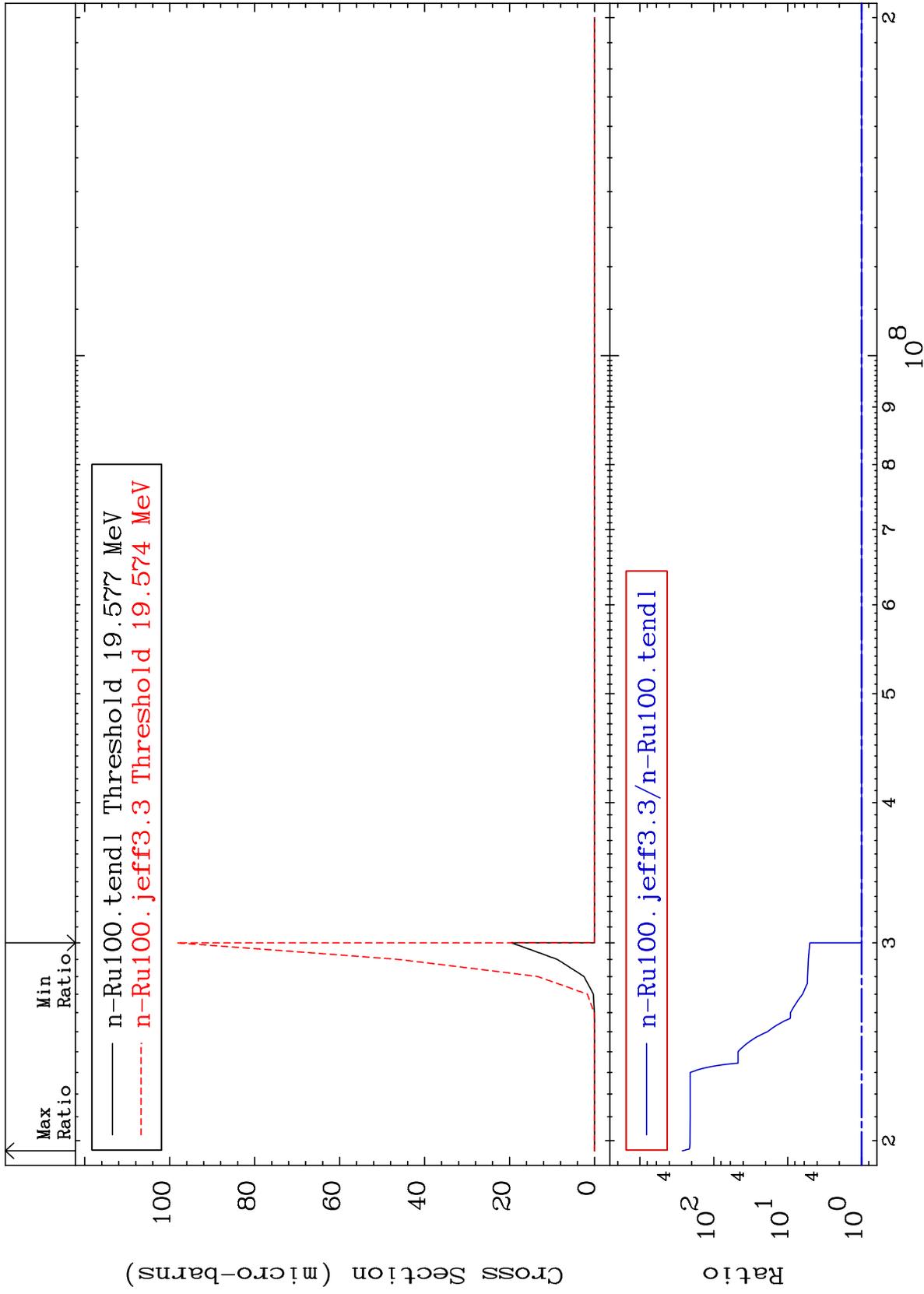


MAT 4437

(n,3n)  $\alpha$   
Cross Section

44-Ru-100

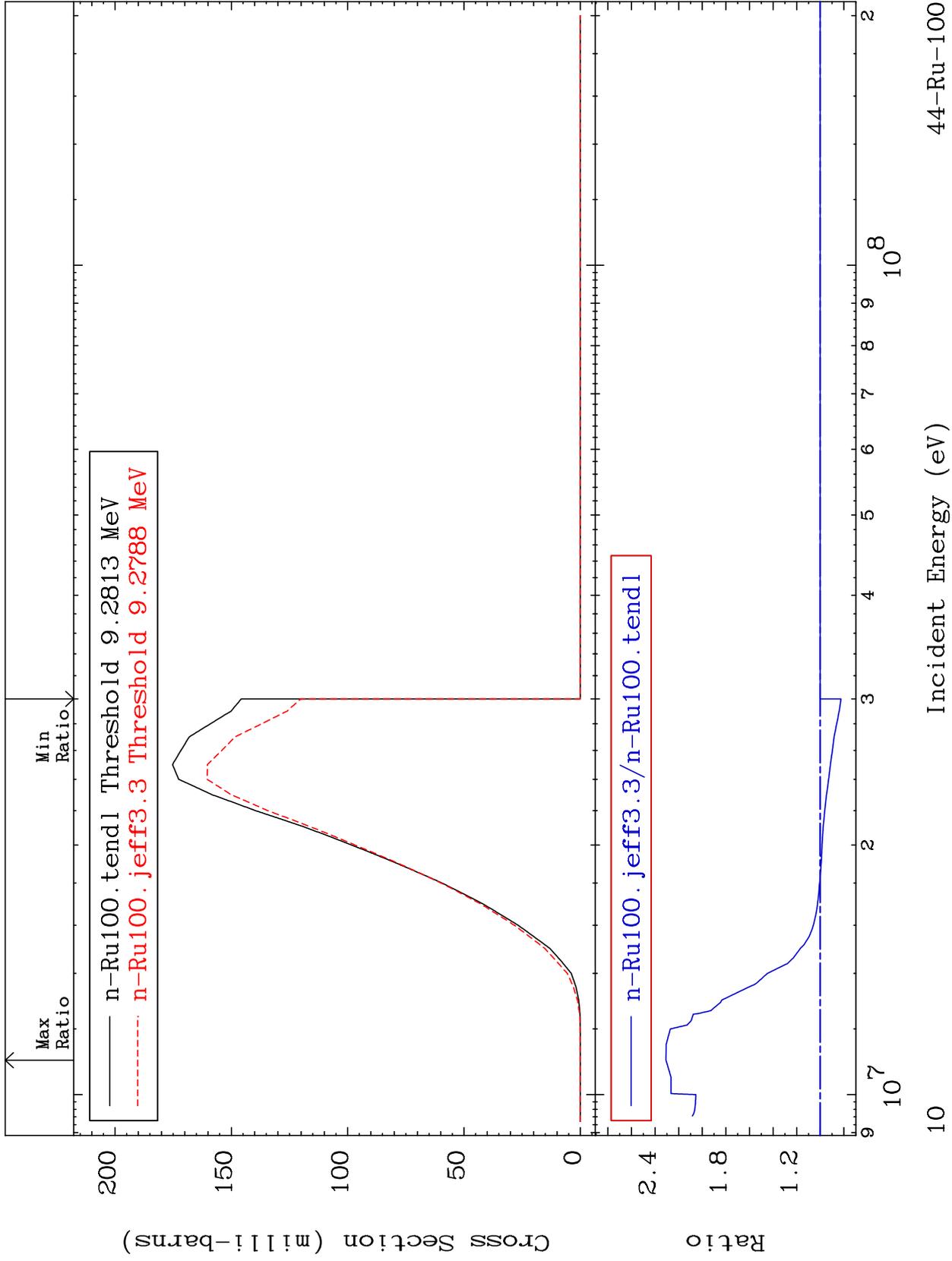
0.000 To 9999. %



MAT 4437

(n,n') p  
Cross Section

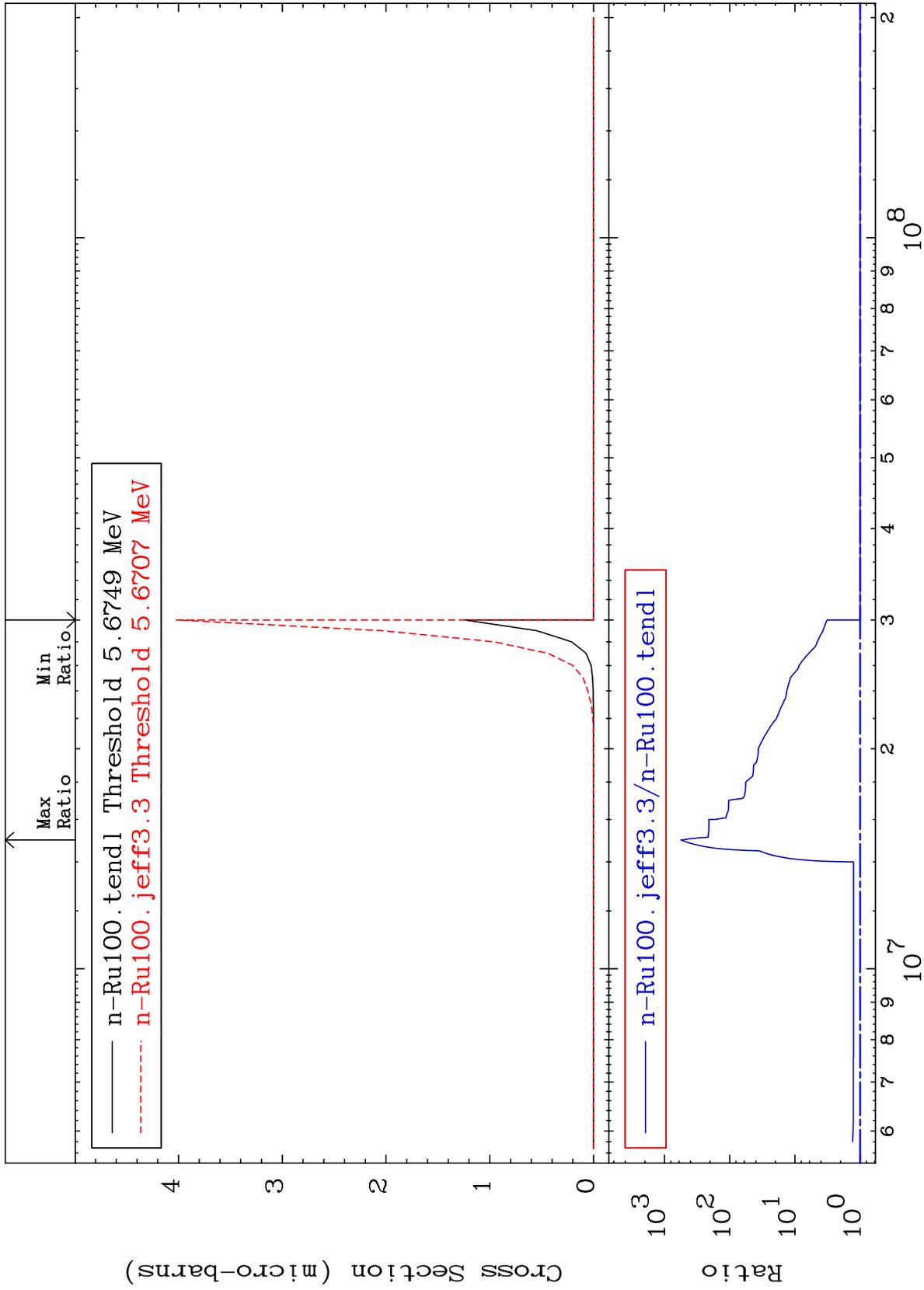
44-Ru-100  
-17.54 To 130.8 %



MAT 4437

(n, n')  $2\alpha$   
Cross Section

44-Ru-100  
To 9999. %



MAT 4437

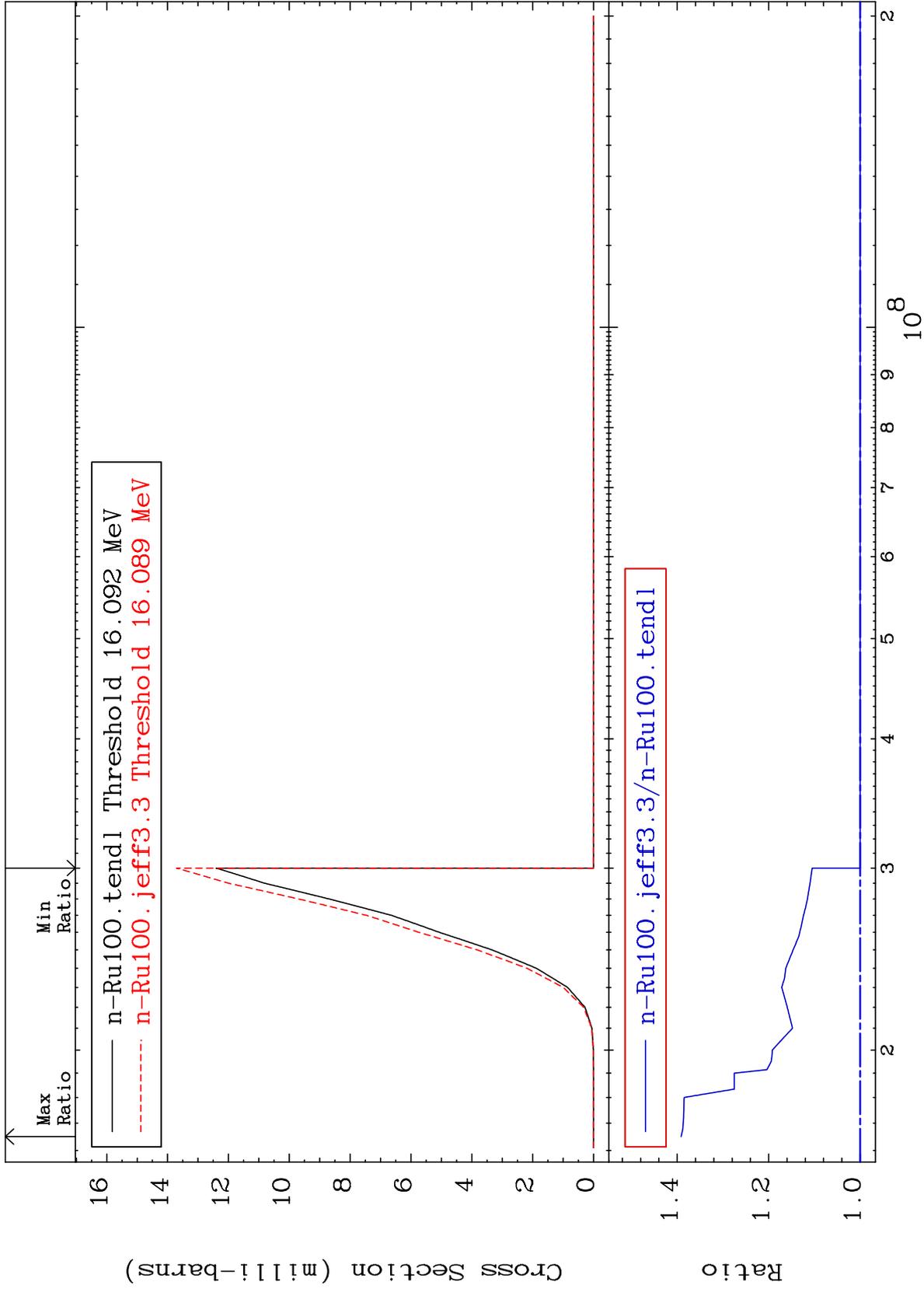
(n,n') d

44-Ru-100

Cross Section

0.000

To 39.13 %



12

44-Ru-100

44-Ru-100

MAT 4437

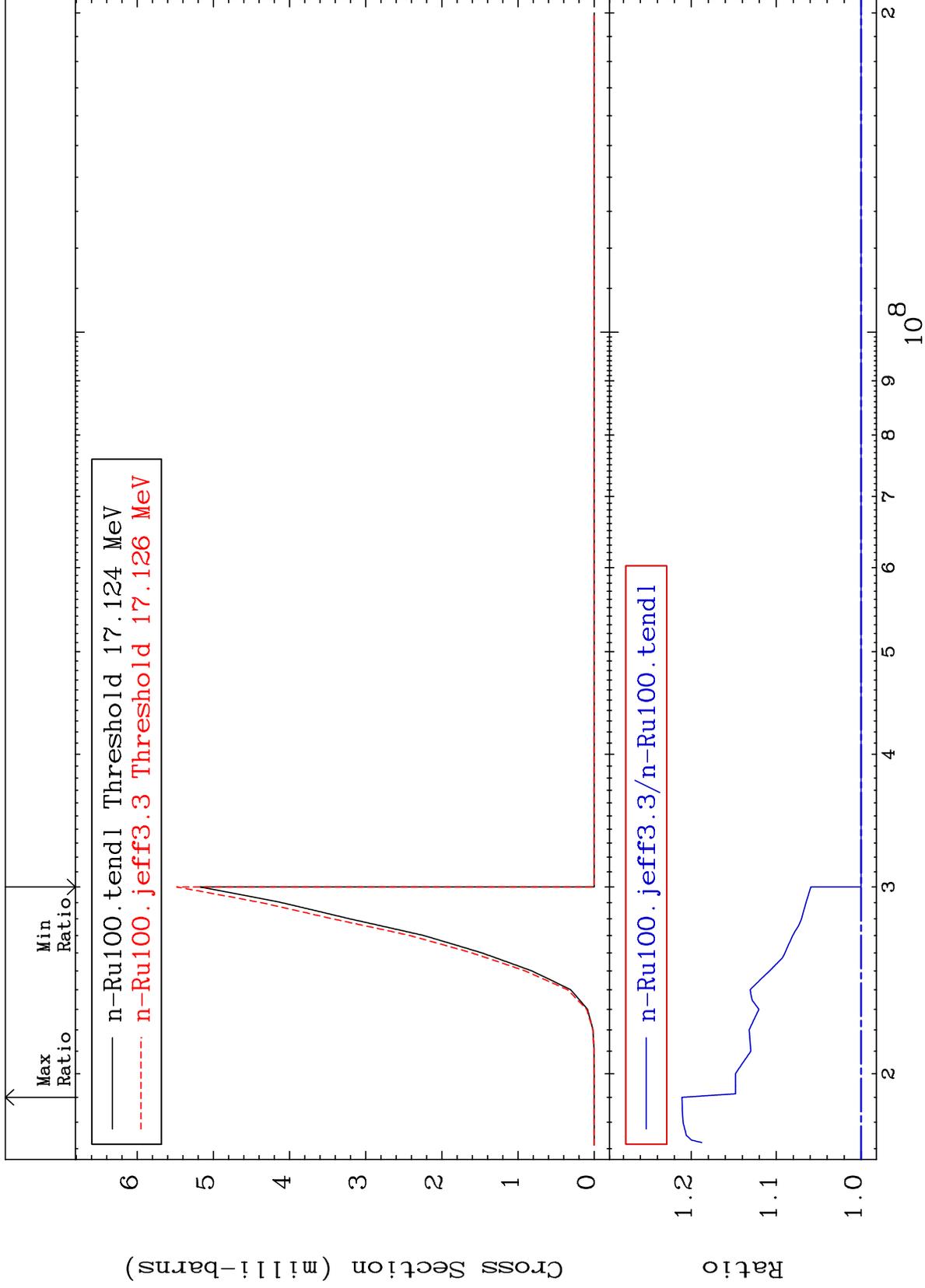
(n,n') t

44-Ru-100

Cross Section

0.000

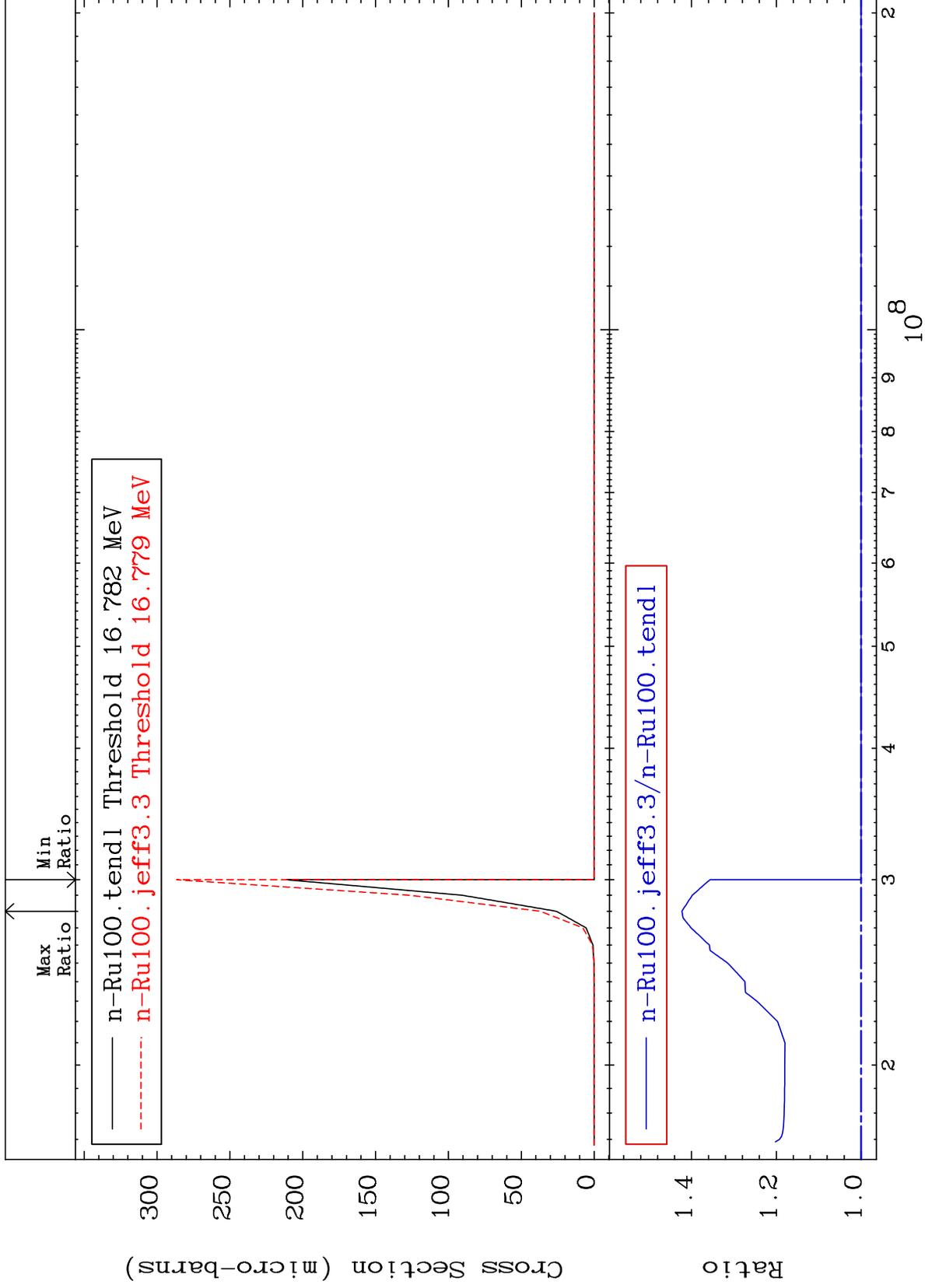
To 21.11 %



MAT 4437

(n, n') He-3  
Cross Section

44-Ru-100  
To 42.27 %



MAT 4437

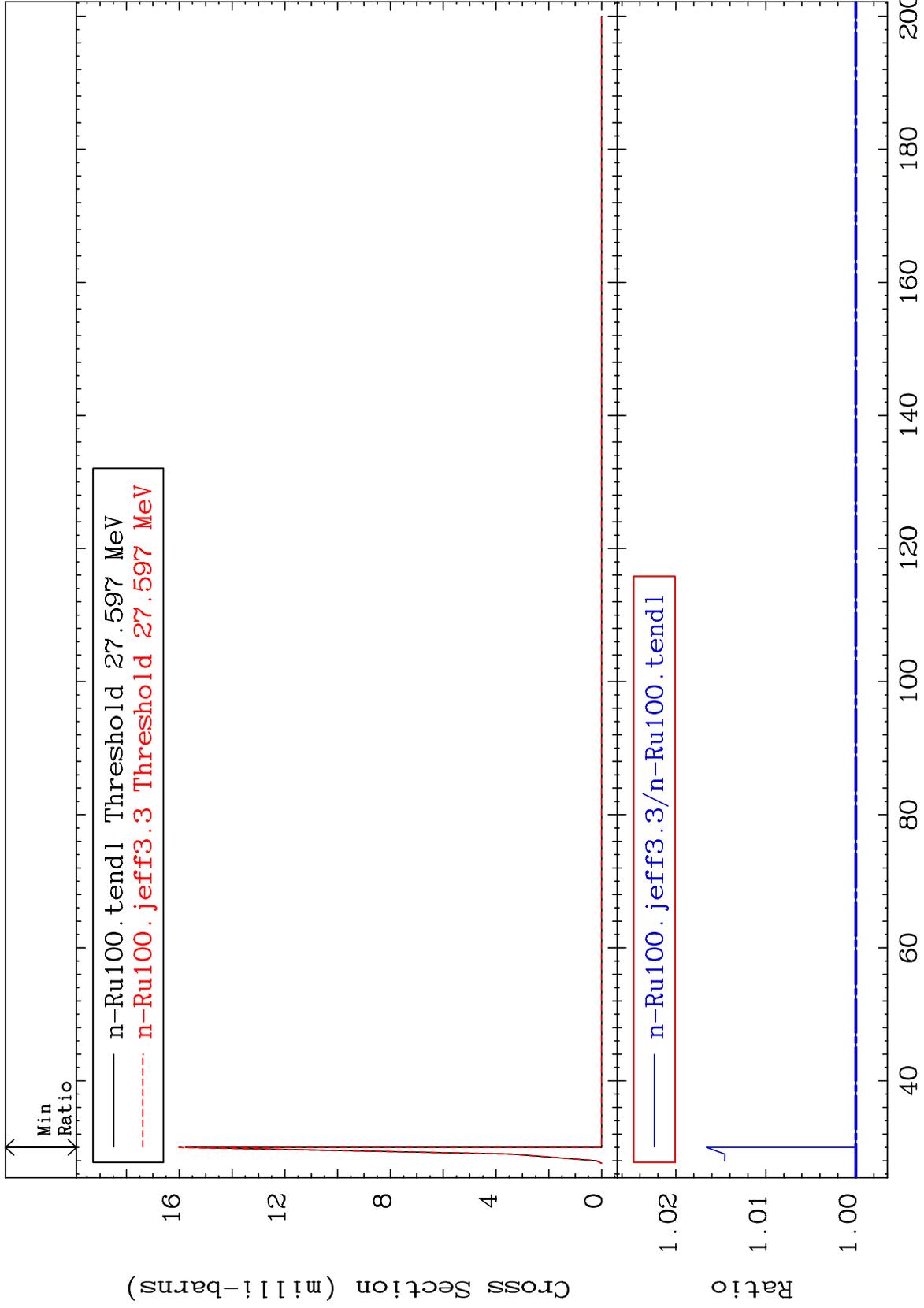
(n,4n)

44-Ru-100

Cross Section

0.000

To 1.660 %



Min  
Ratio

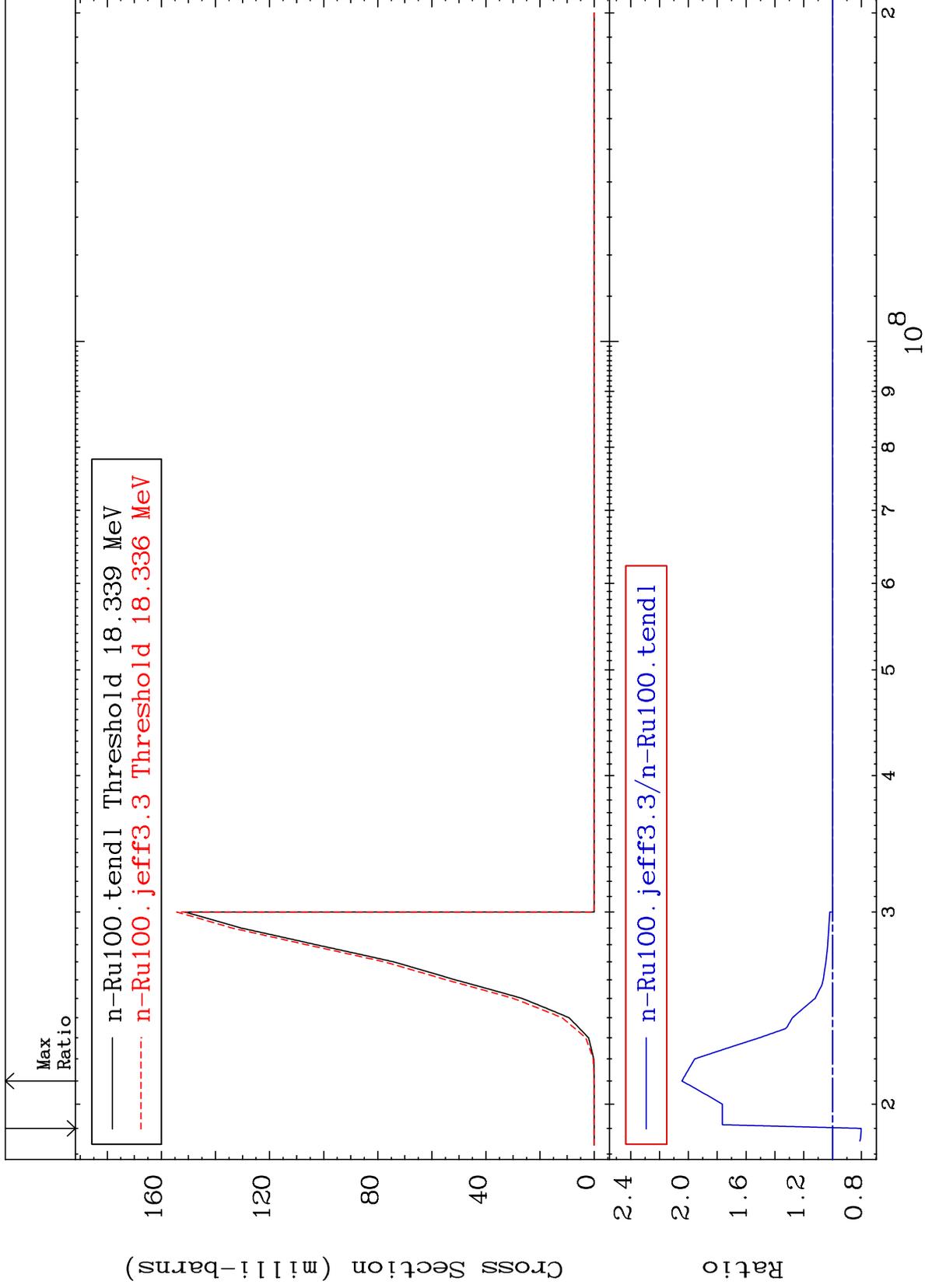
n-Ru100.tendl Threshold 27.597 MeV  
n-Ru100.jeff3.3 Threshold 27.597 MeV

n-Ru100.jeff3.3/n-Ru100.tendl

MAT 4437

(n,2n) p  
Cross Section

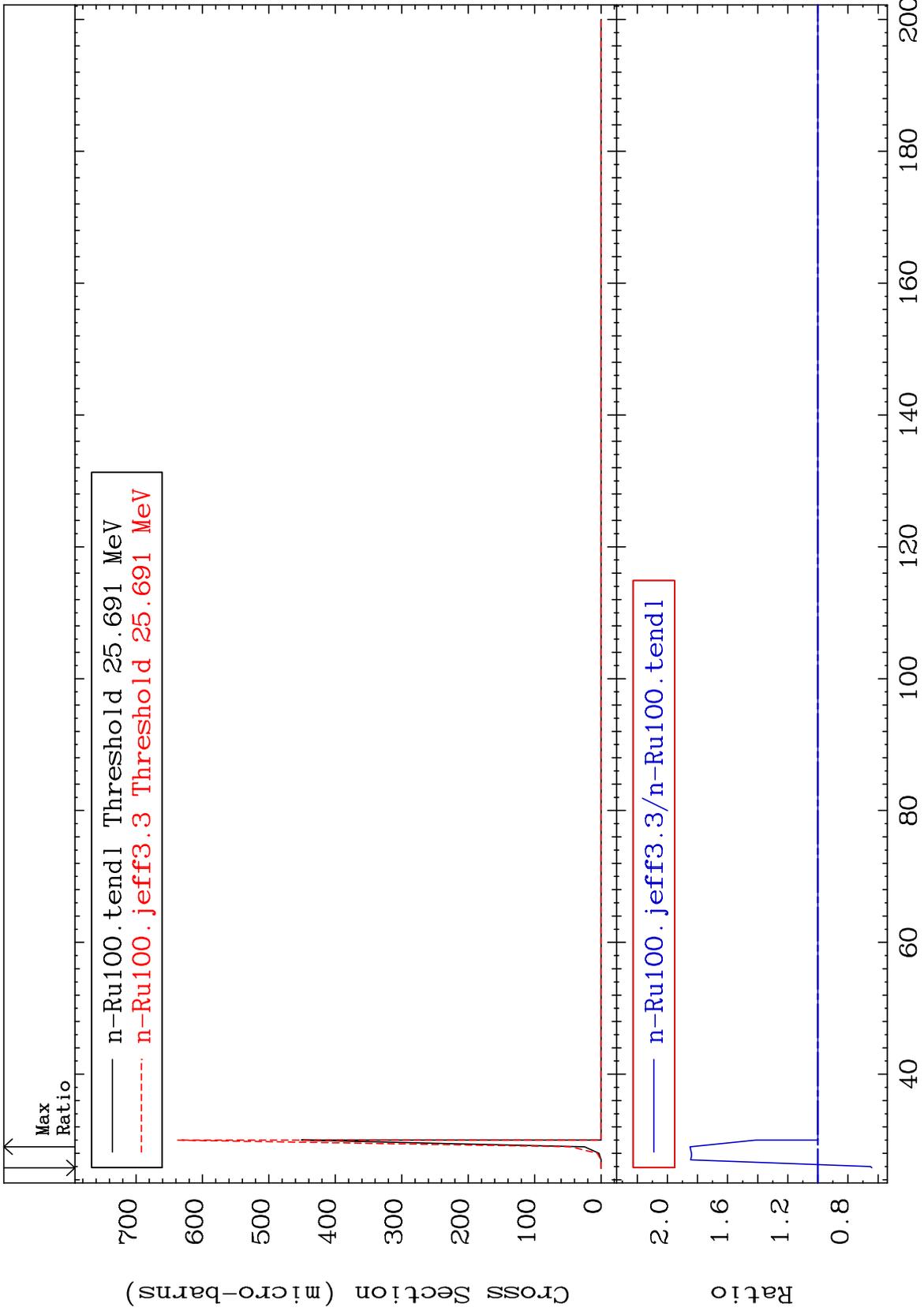
44-Ru-100  
-19.79 To 104.5 %



MAT 4437

(n,3n) p  
Cross Section

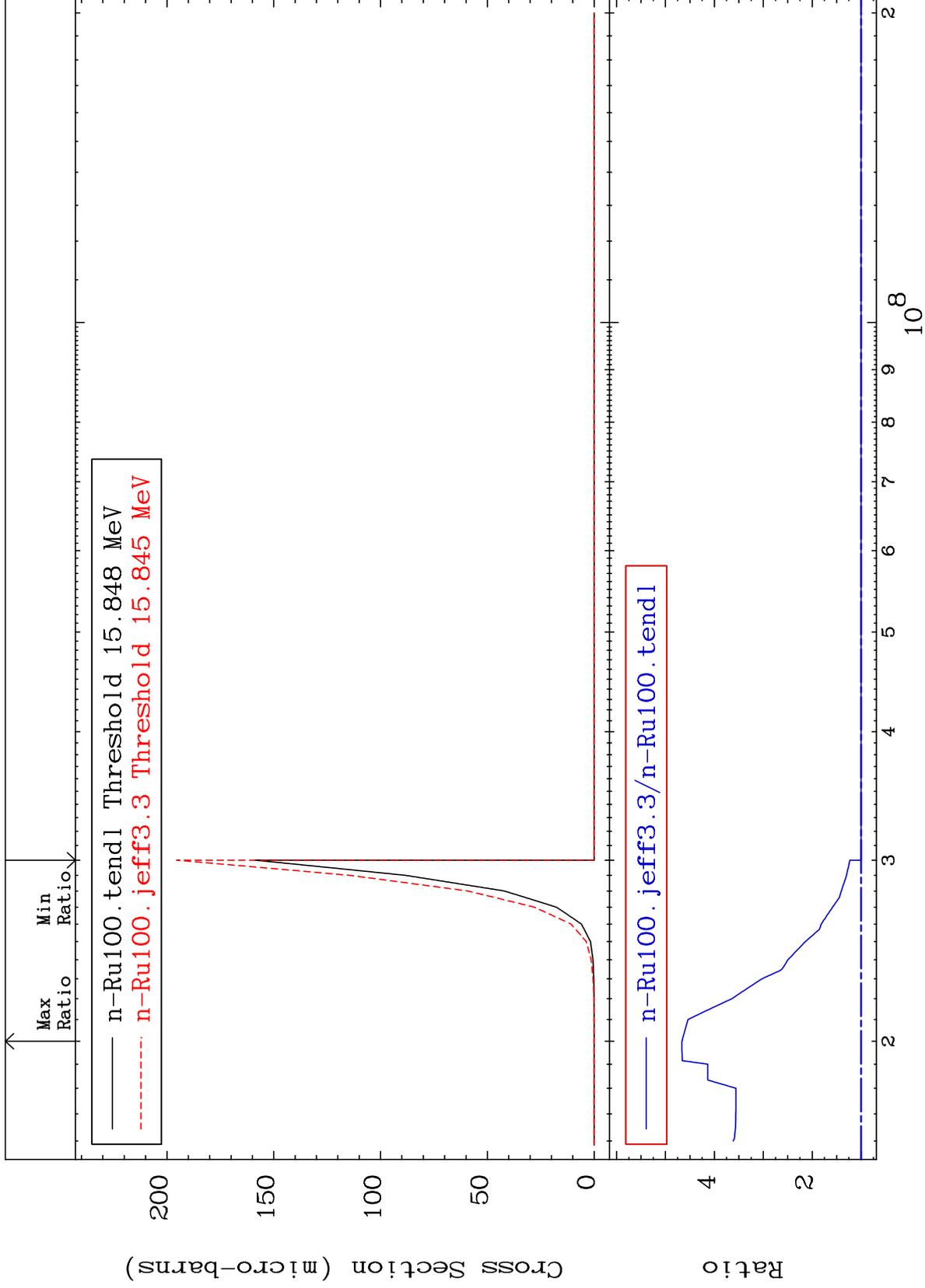
44-Ru-100  
-35.84 To 84.92 %

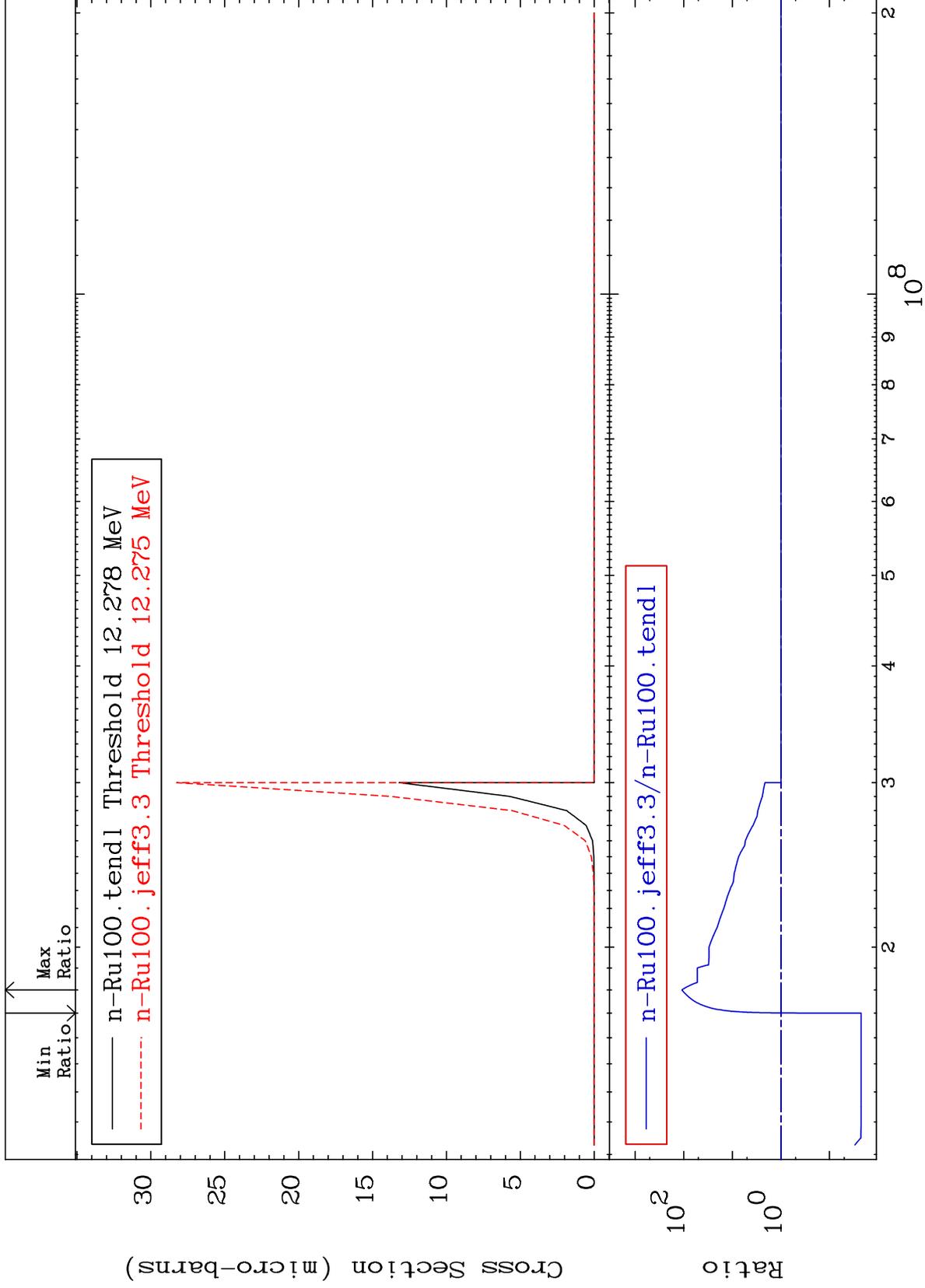


MAT 4437

(n,2n) p  
Cross Section

44-Ru-100  
To 366.4 %  
0.000

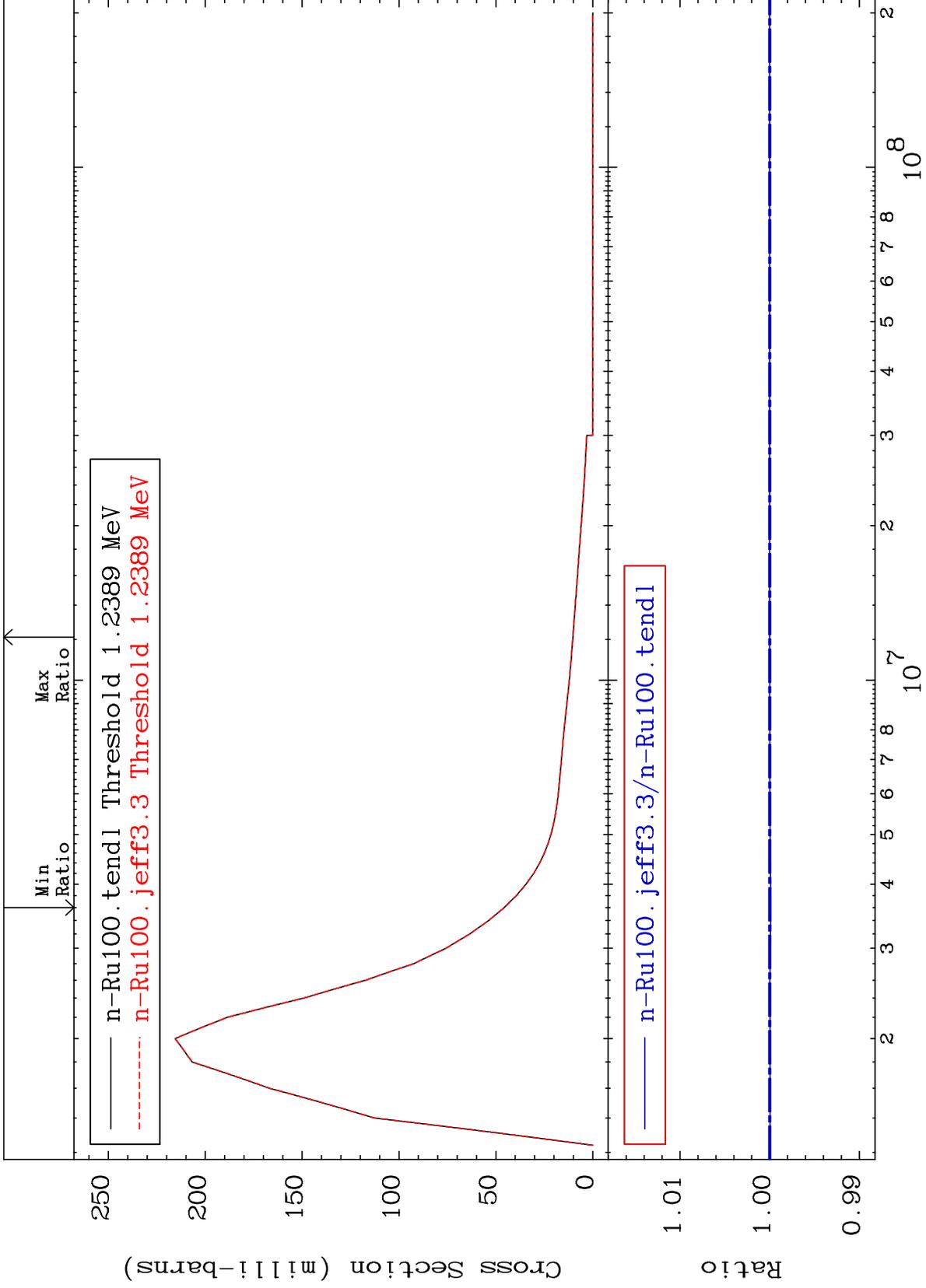




MAT 4437

MT= 53 (n,n') Level  
Cross Section

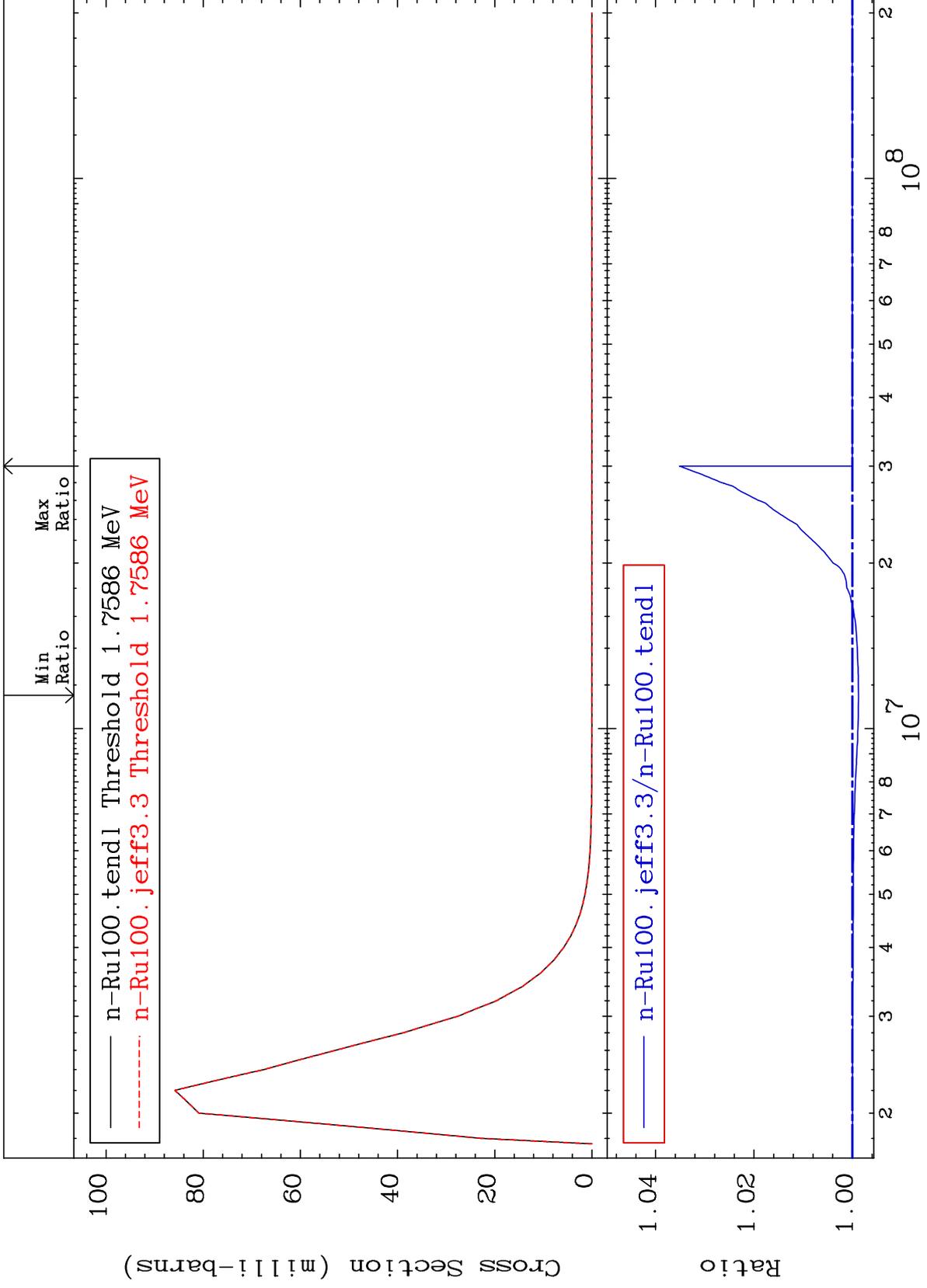
44-Ru-100  
-0.011 To 0.000 %



MAT 4437

MT= 55 (n,n') Level  
Cross Section

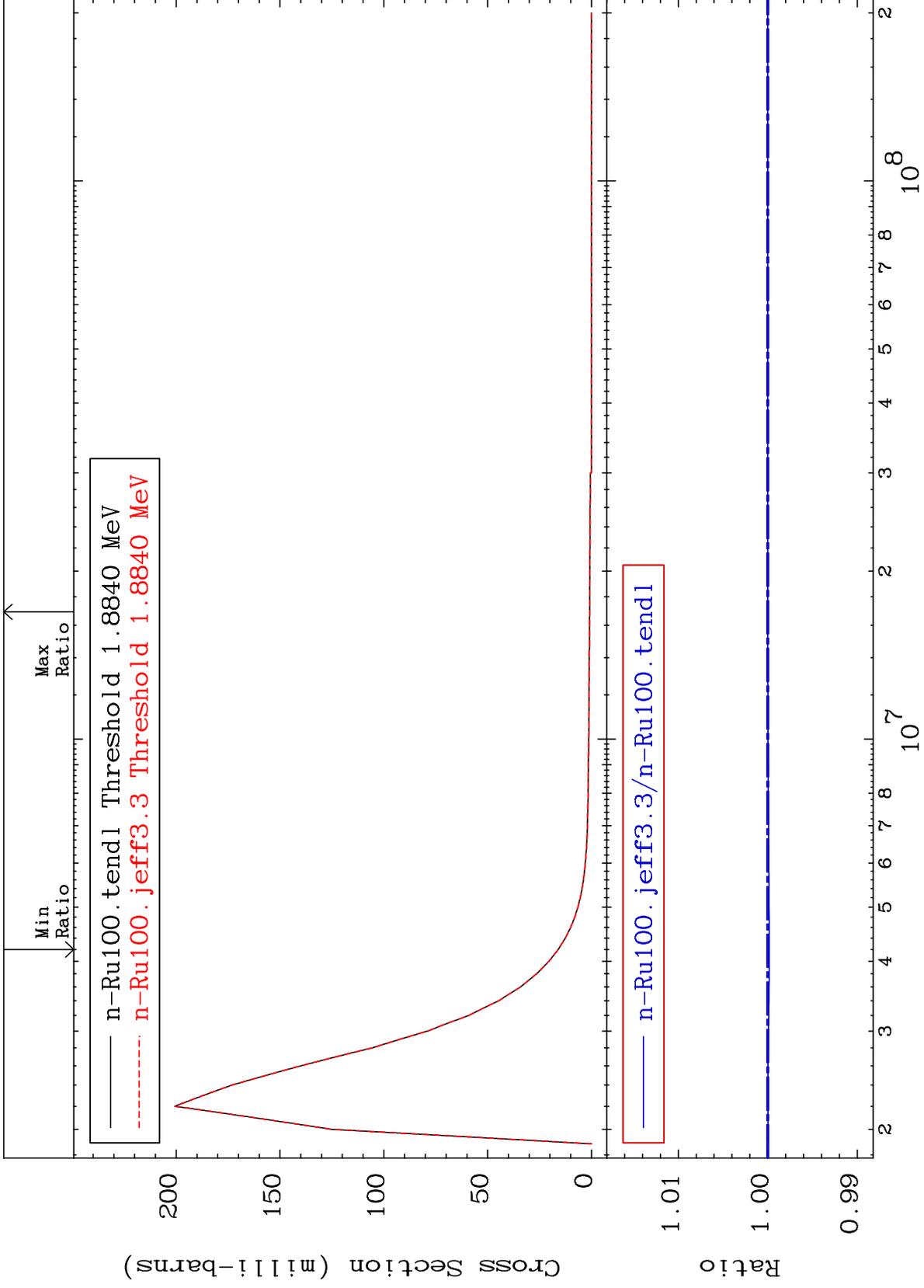
44-Ru-100  
-0.126 To 3.513 %



MAT 4437

MT= 56 (n,n') Level  
Cross Section

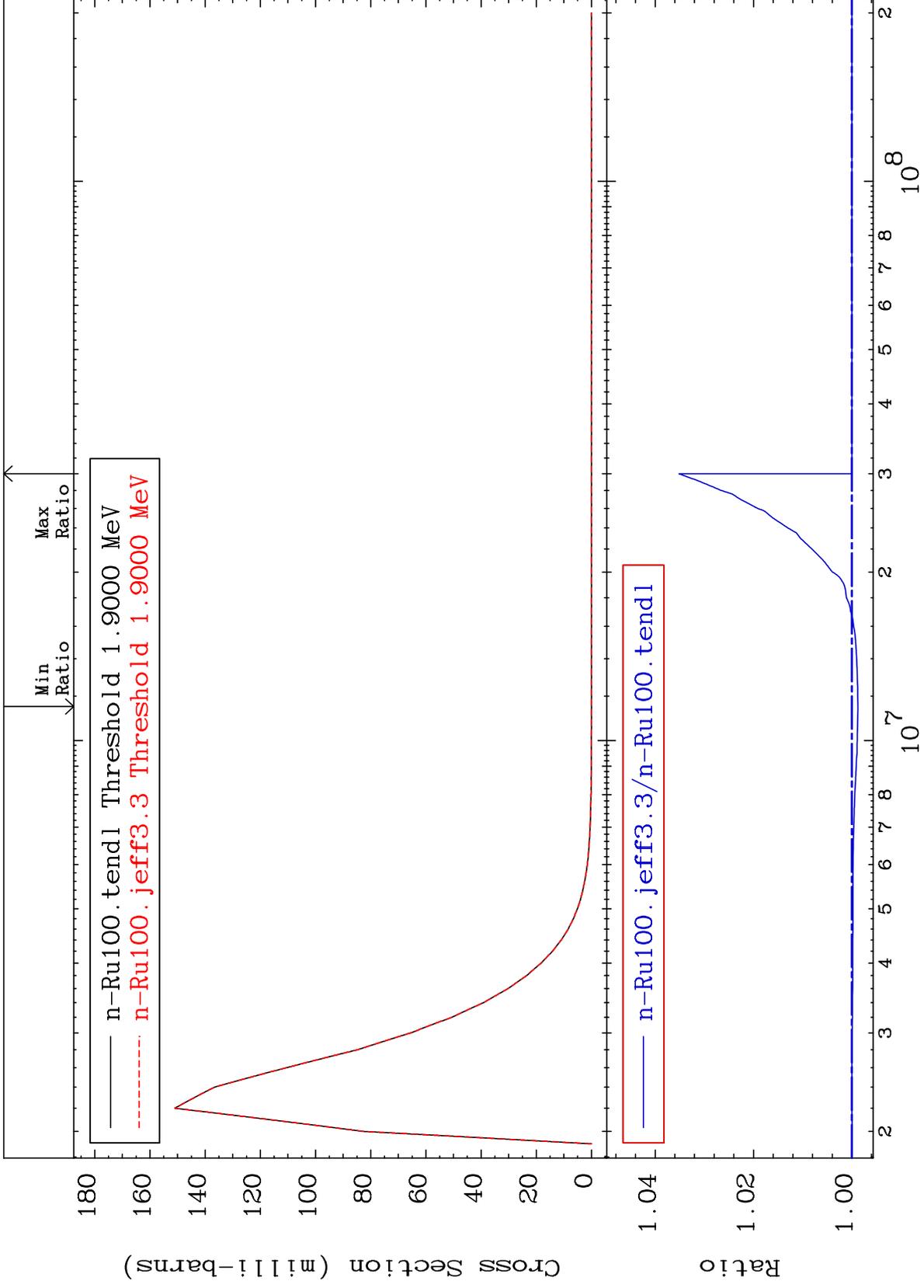
44-Ru-100  
-0.016 To 0.000 %



MAT 4437

MT= 57 (n,n') Level  
Cross Section

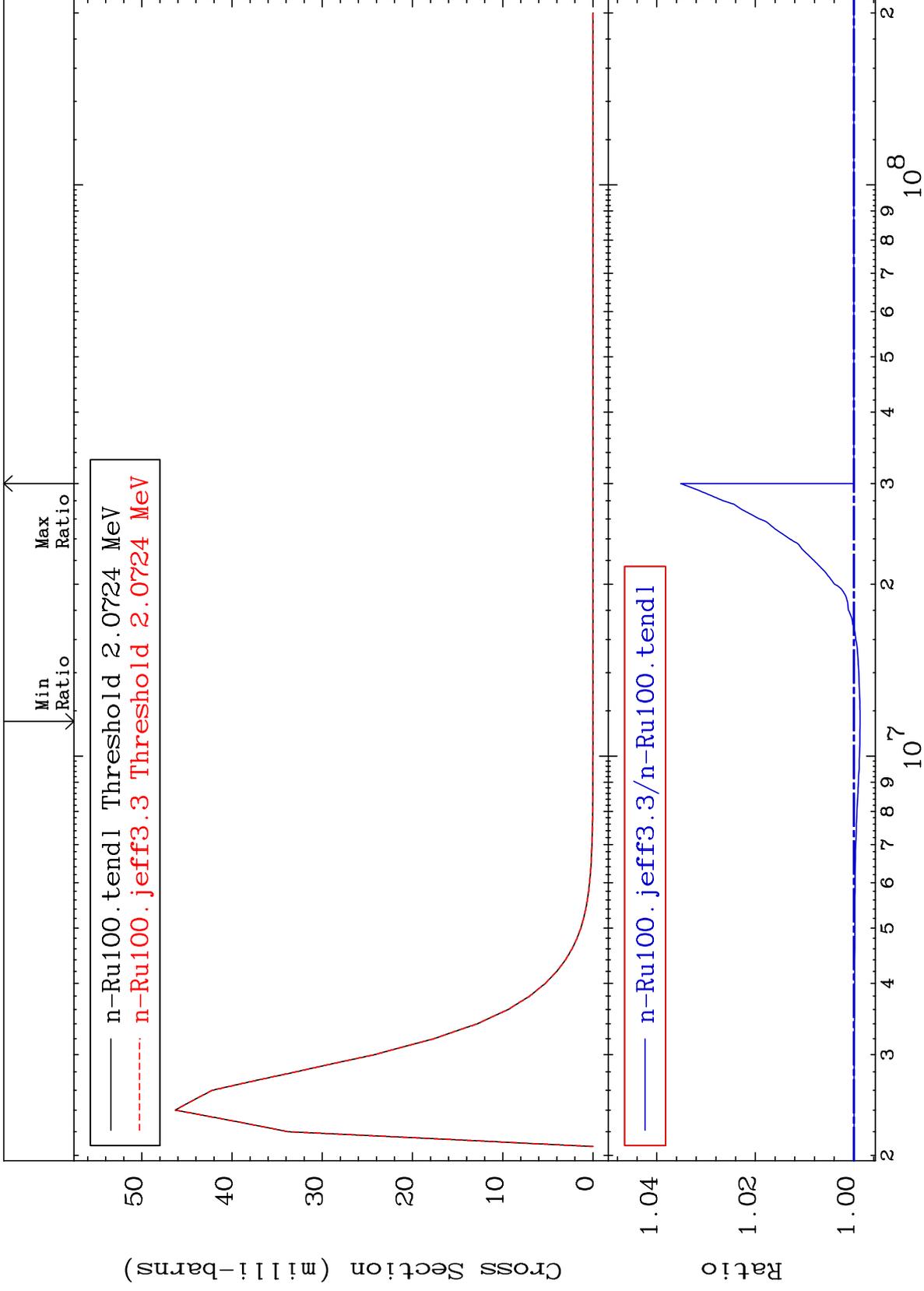
44-Ru-100  
-0.123 To 3.514 %



MAT 4437

MT= 58 (n,n') Level  
Cross Section

44-Ru-100  
-0.126 To 3.513 %



24

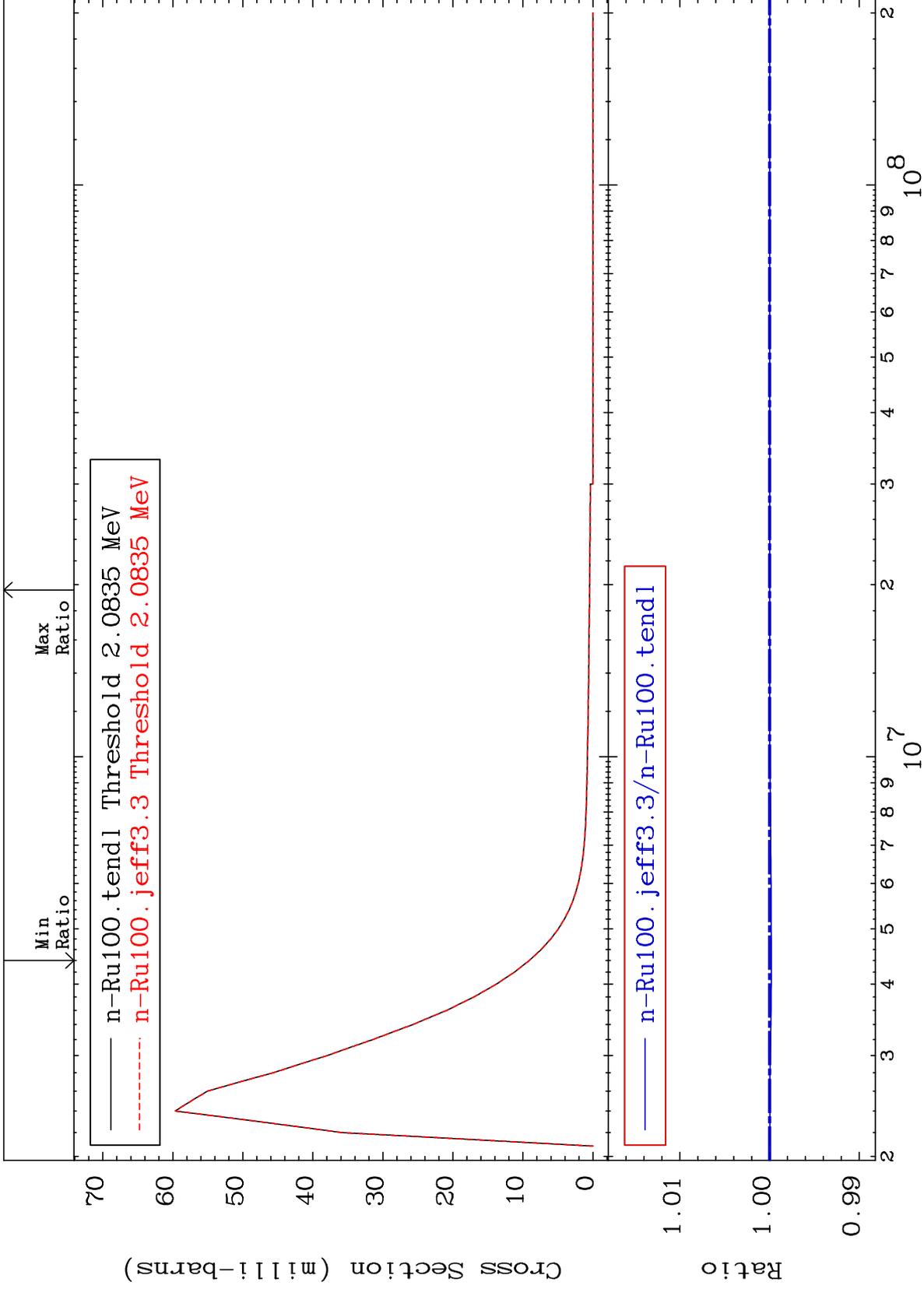
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 59 (n,n') Level  
Cross Section

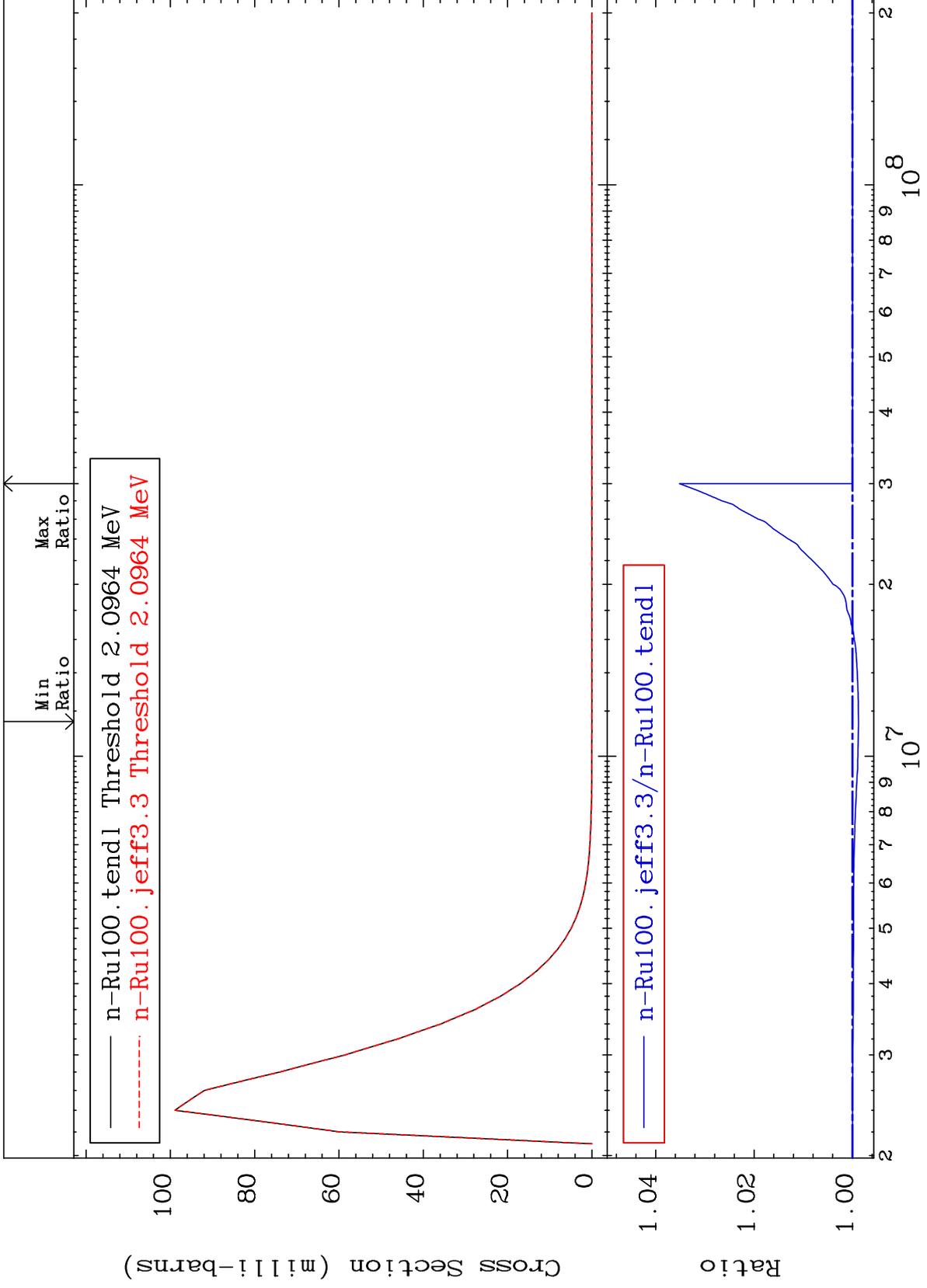
44-Ru-100  
-0.019 To 0.000 %



MAT 4437

MT= 60 (n,n') Level  
Cross Section

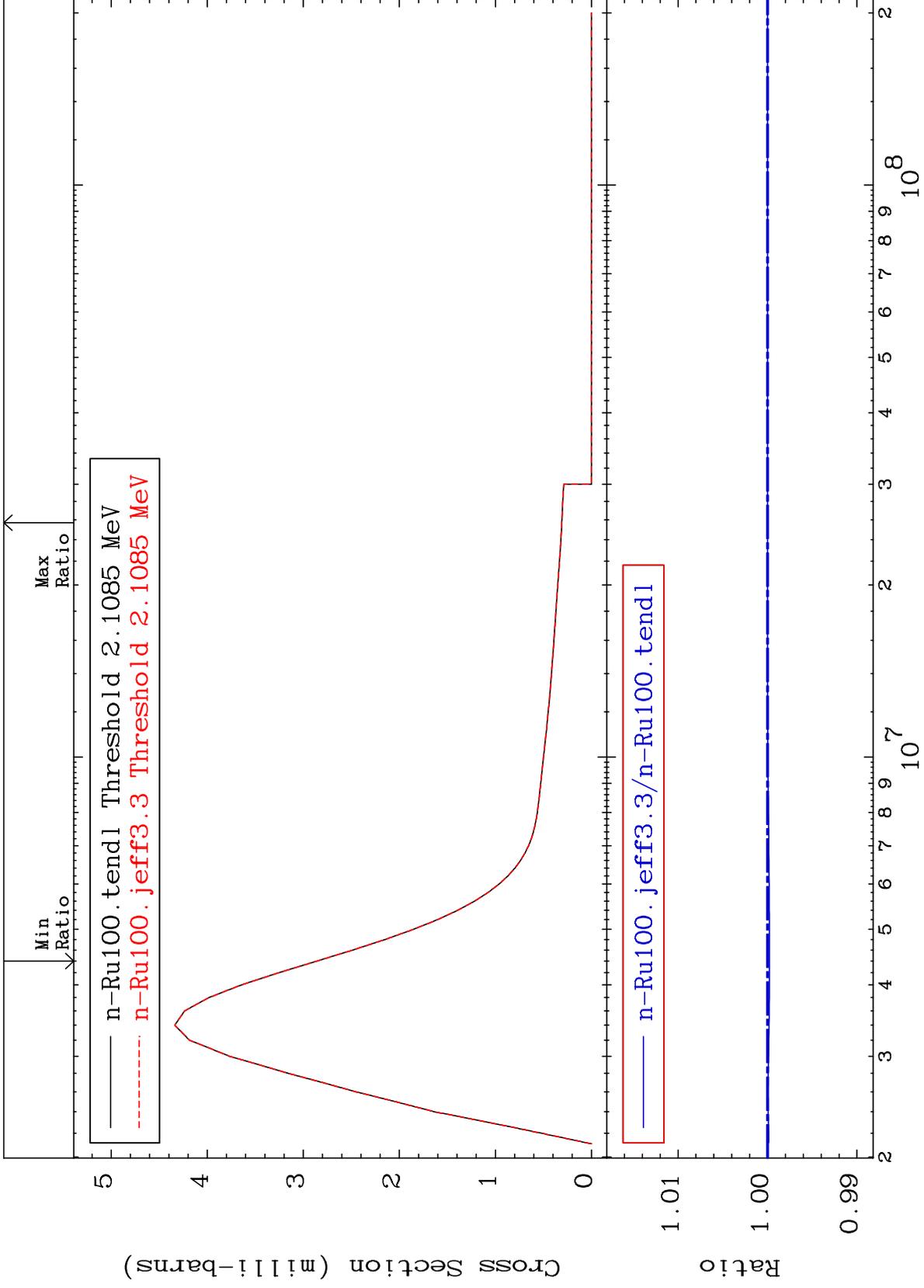
44-Ru-100  
-0.123 To 3.514 %



MAT 4437

MT= 61 (n,n') Level  
Cross Section

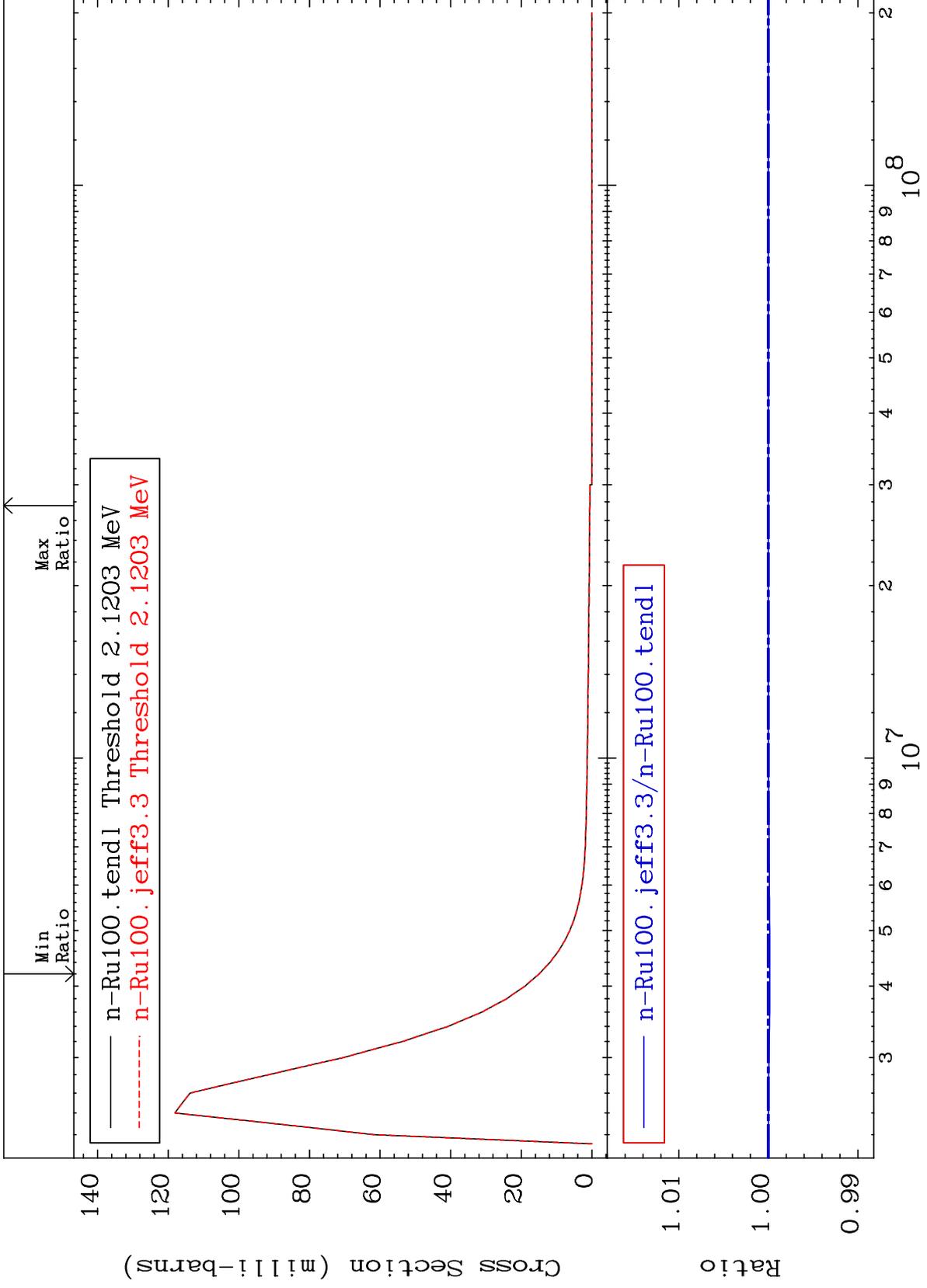
44-Ru-100  
-0.022 To 0.000 %



MAT 4437

MT= 62 (n, n') Level  
Cross Section

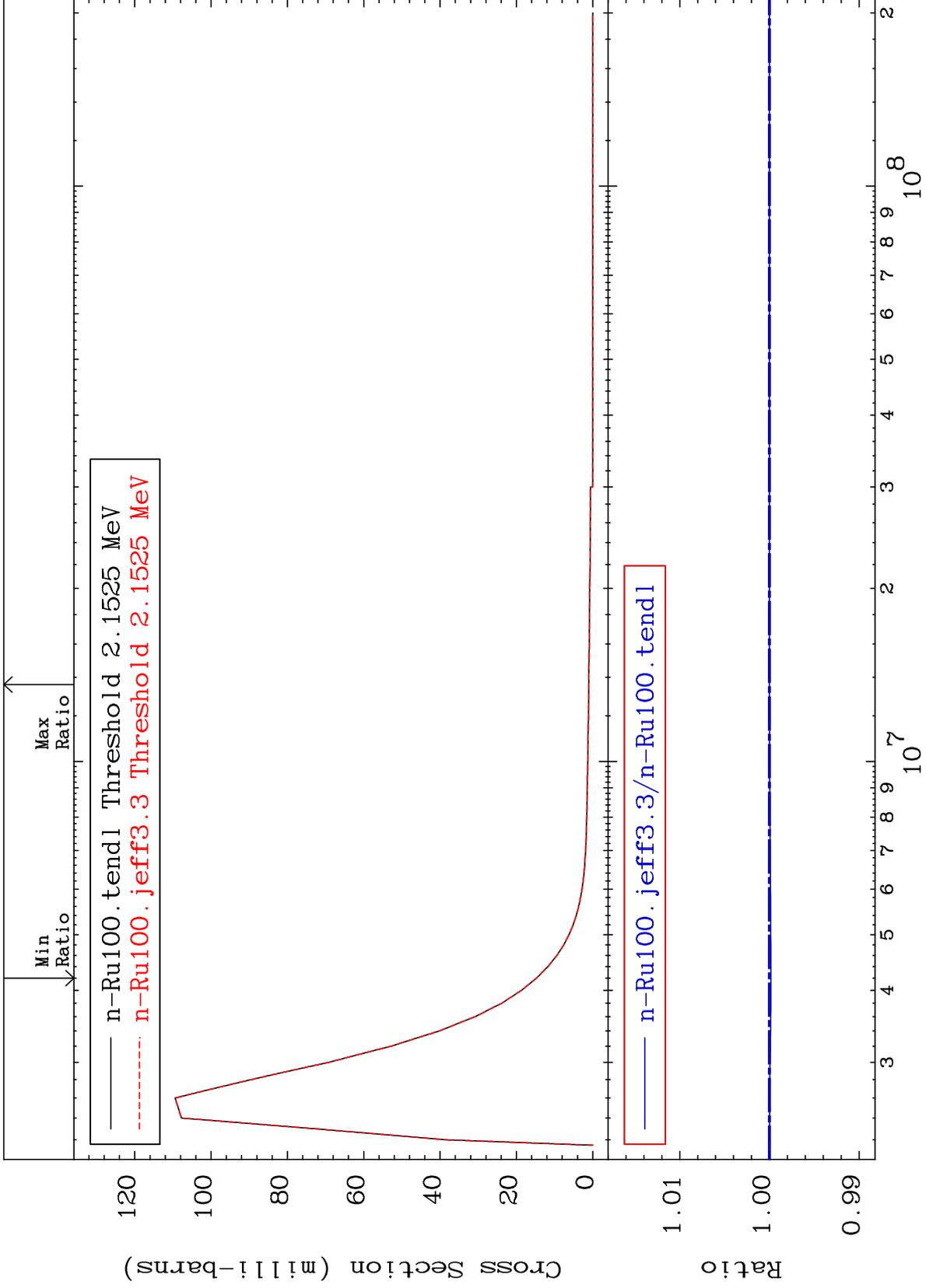
44-Ru-100  
-0.016 To 0.000 %



MAT 4437

MT= 63 (n,n') Level  
Cross Section

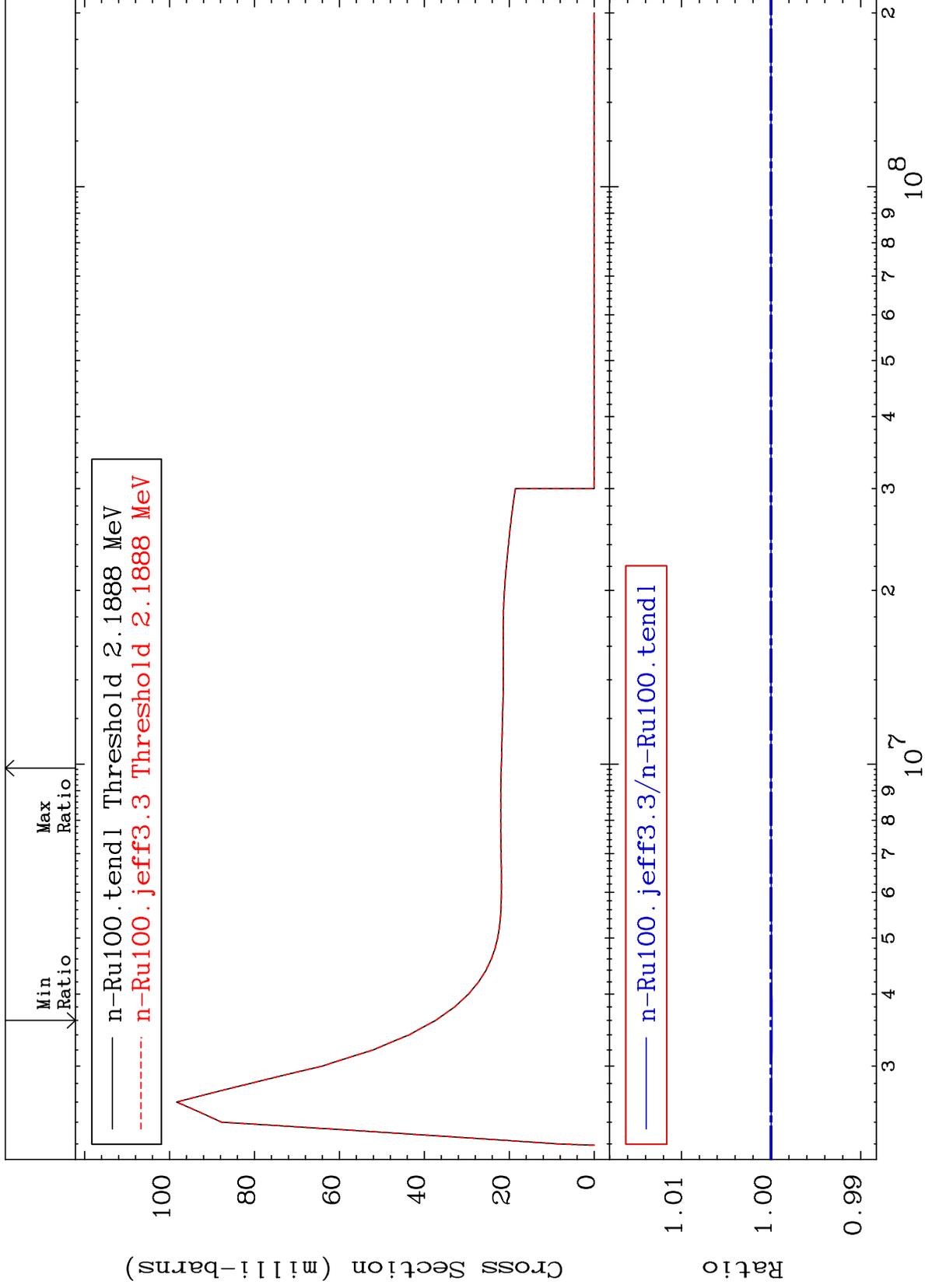
44-Ru-100  
-0.016 To 0.000 %



MAT 4437

MT= 64 (n,n') Level  
Cross Section

44-Ru-100  
-0.012 To 0.000 %



30

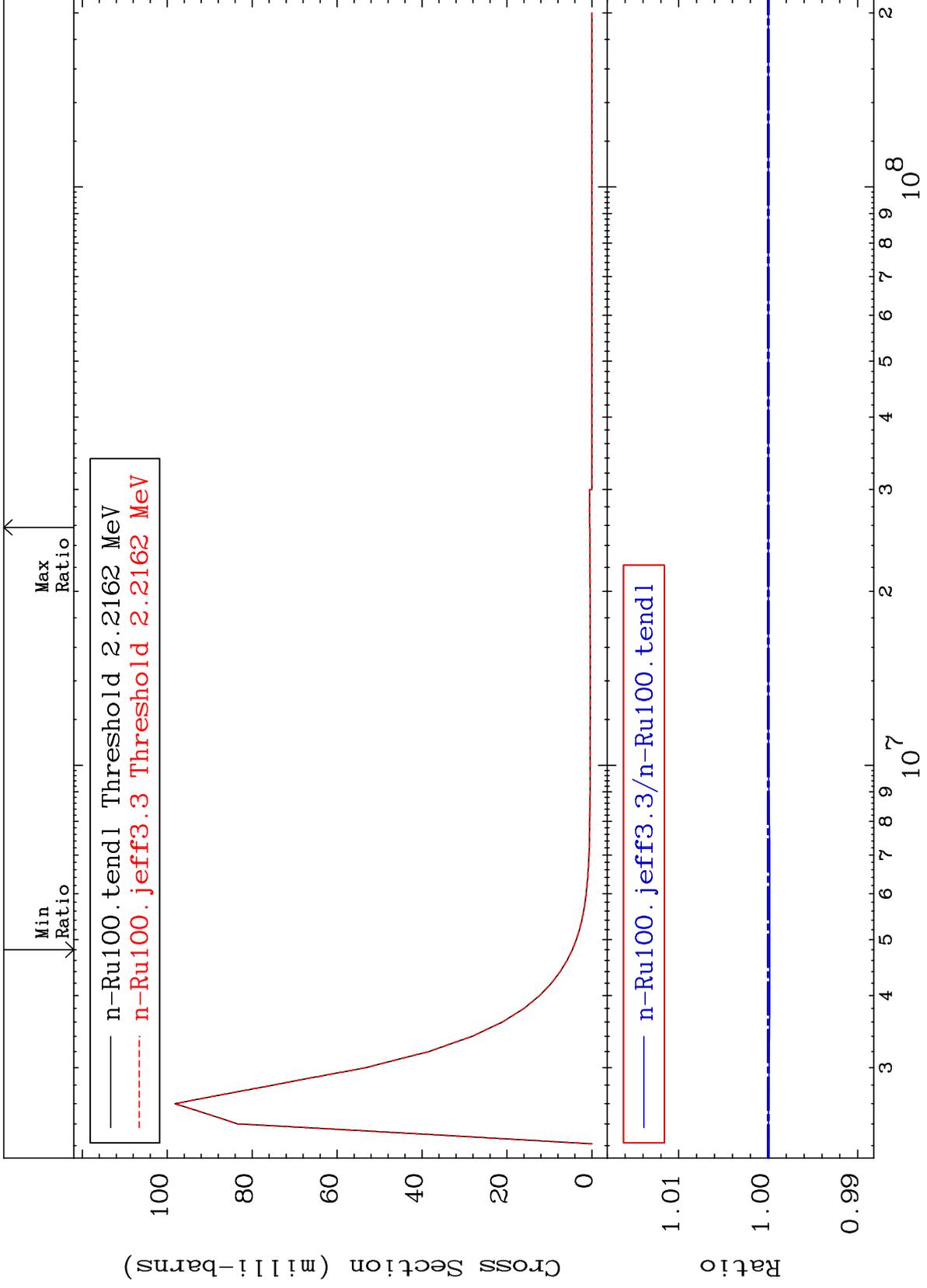
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 65 (n, n') Level  
Cross Section

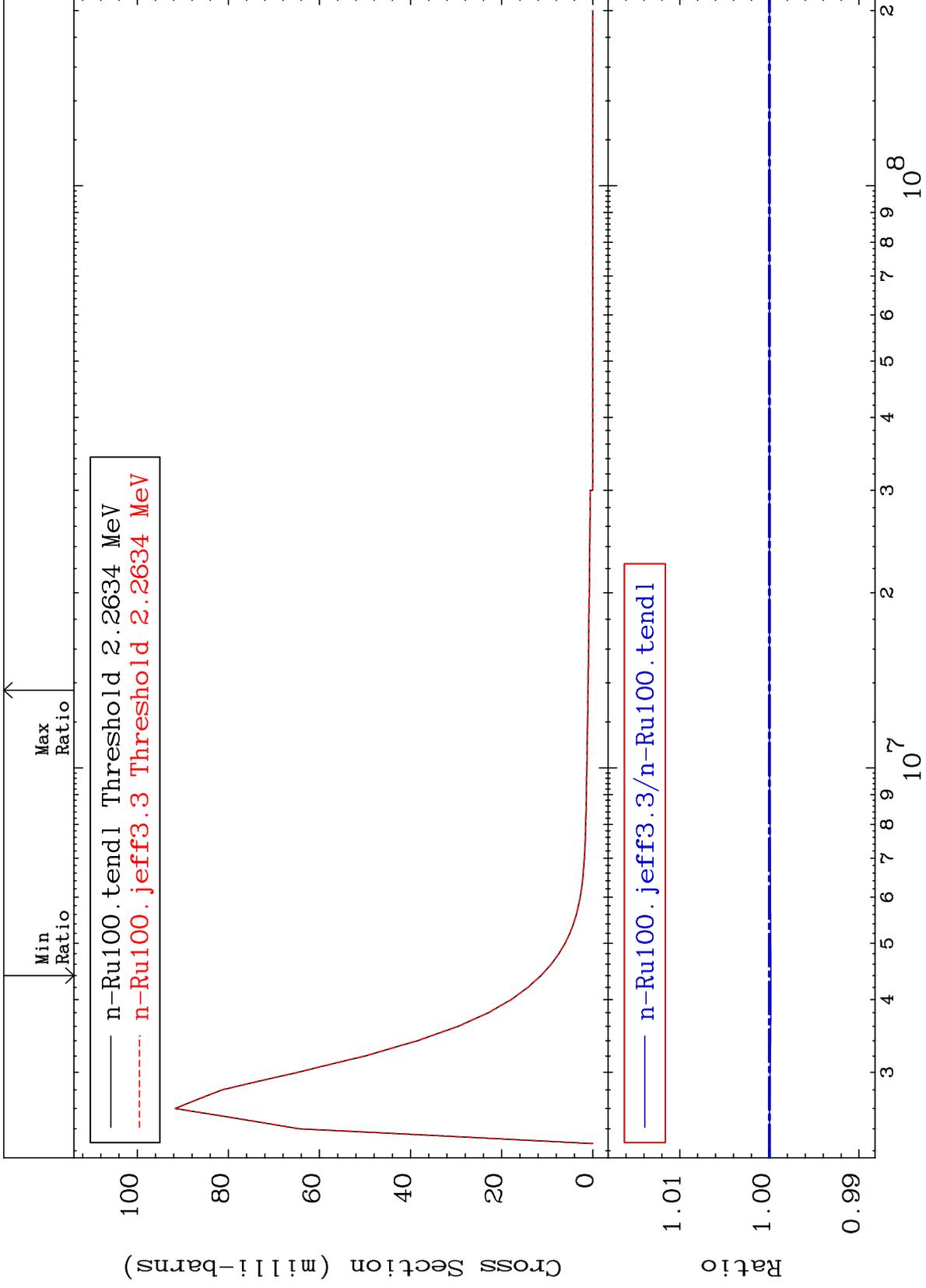
44-Ru-100  
-0.017 To 0.000 %



MAT 4437

MT= 66 (n,n') Level  
Cross Section

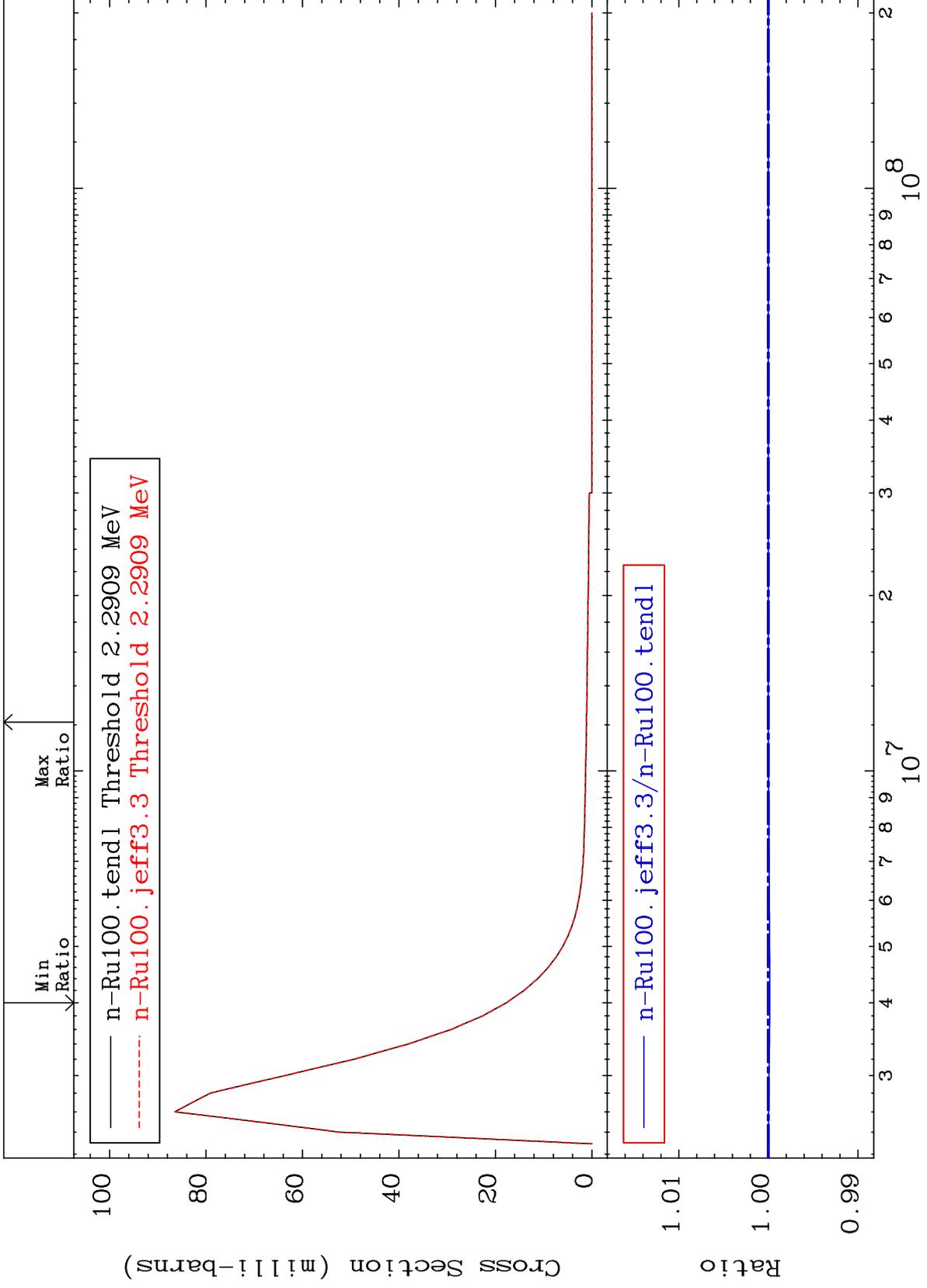
44-Ru-100  
-0.017 To 0.000 %



MAT 4437

MT= 67 (n,n') Level  
Cross Section

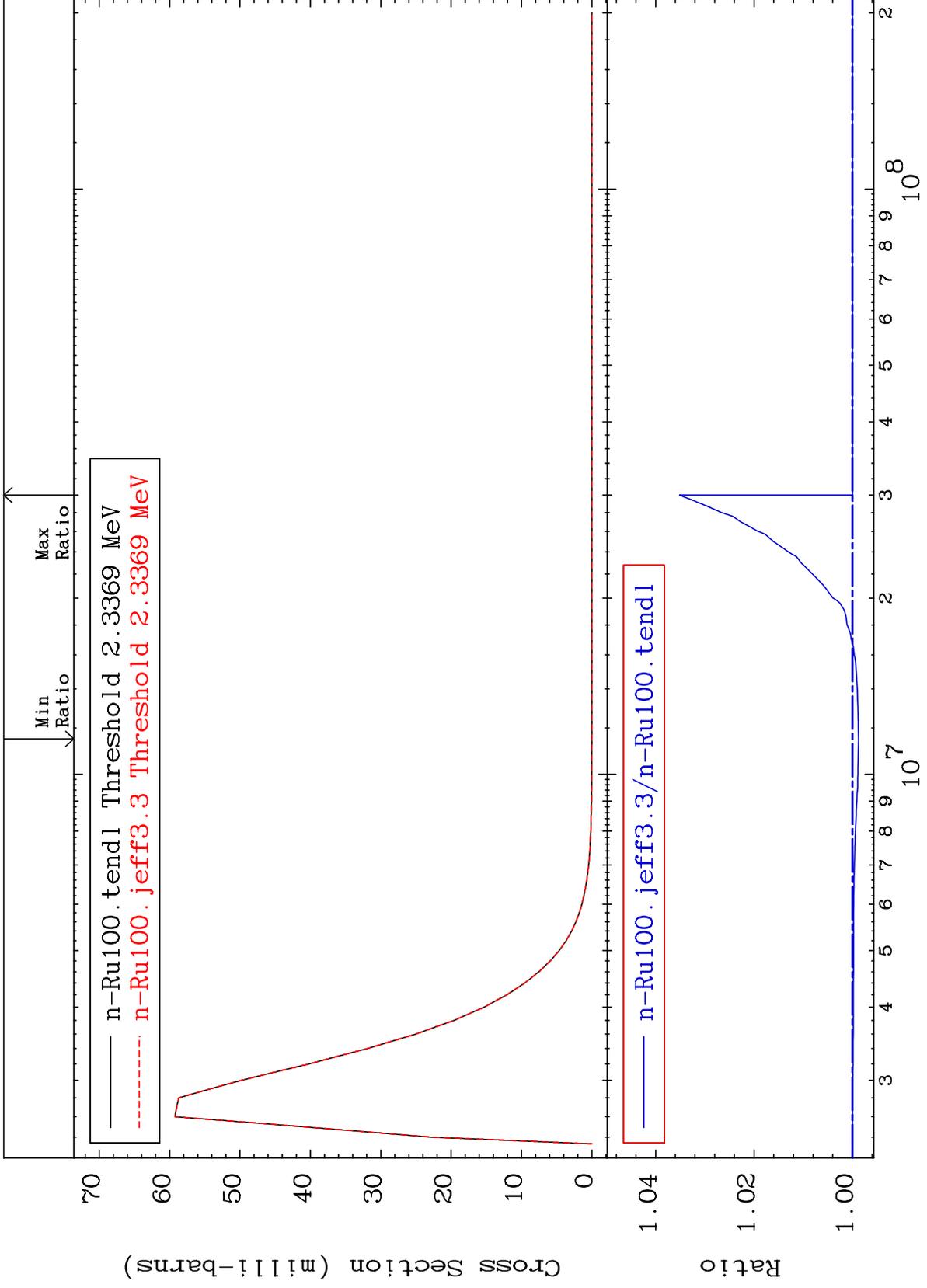
44-Ru-100  
-0.016 To 0.000 %



MAT 4437

MT= 68 (n,n') Level  
Cross Section

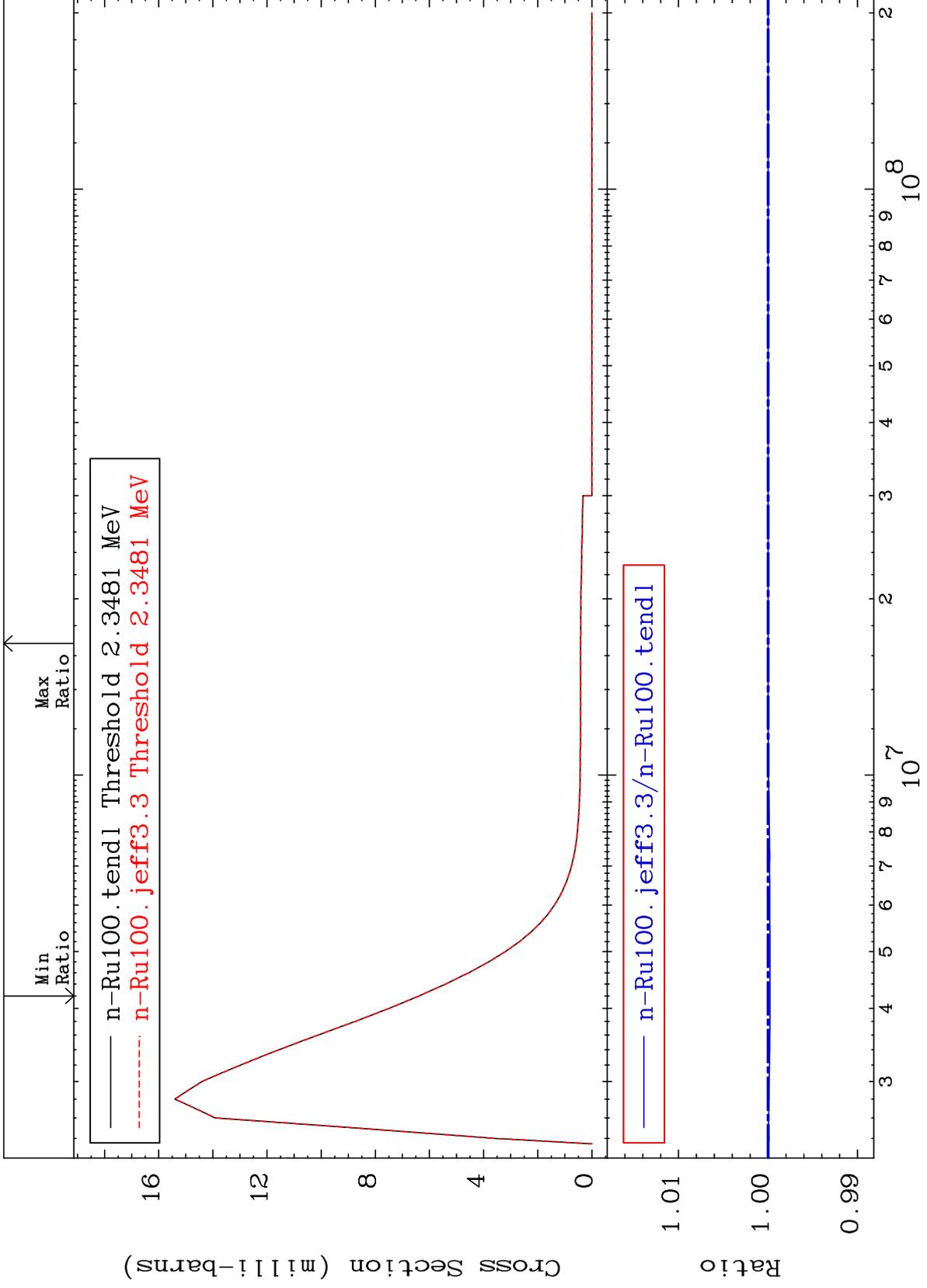
44-Ru-100  
-0.123 To 3.515 %



MAT 4437

MT= 69 (n,n') Level  
Cross Section

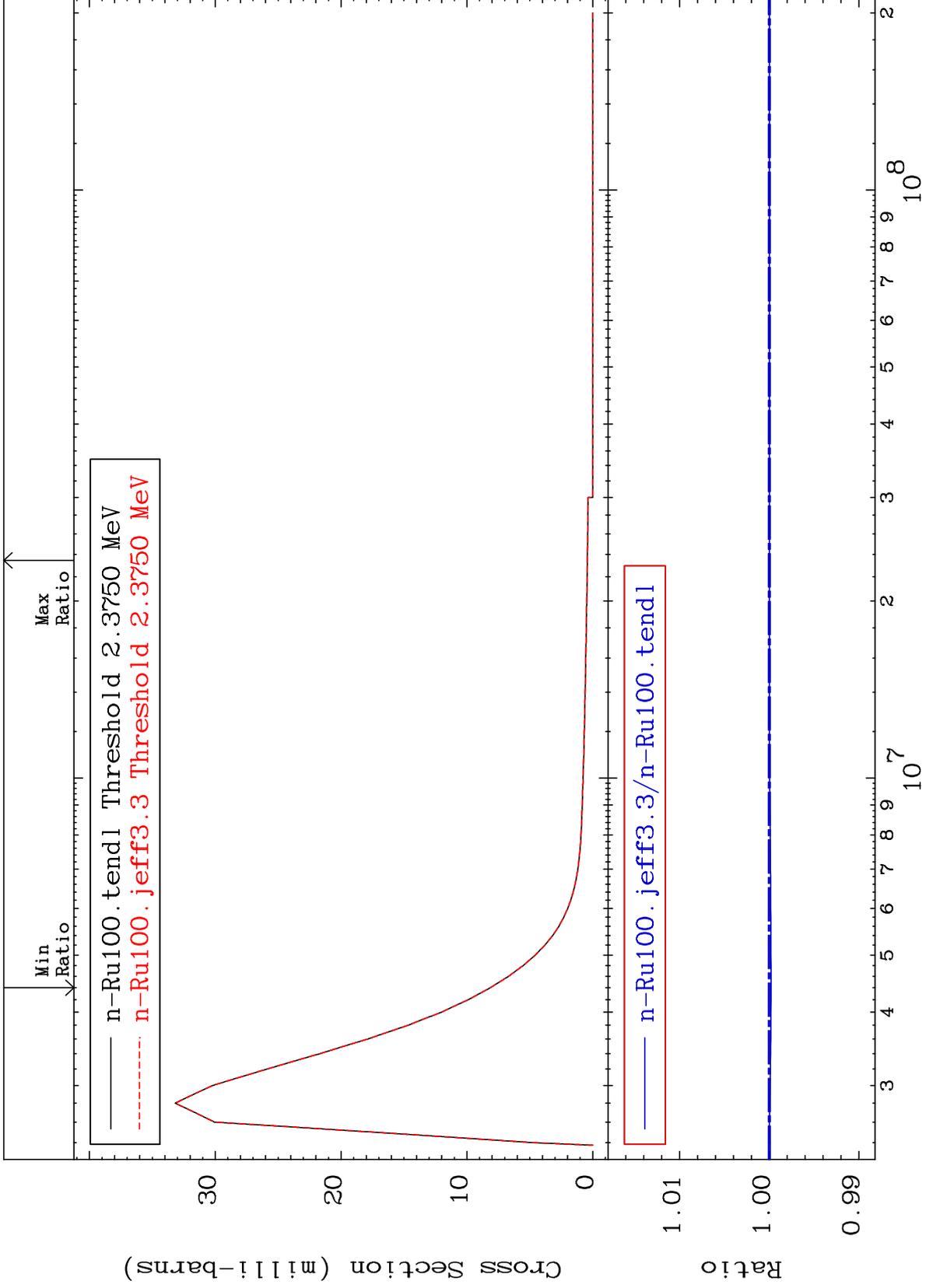
44-Ru-100  
-0.022 To 0.000 %



MAT 4437

MT= 70 (n,n') Level  
Cross Section

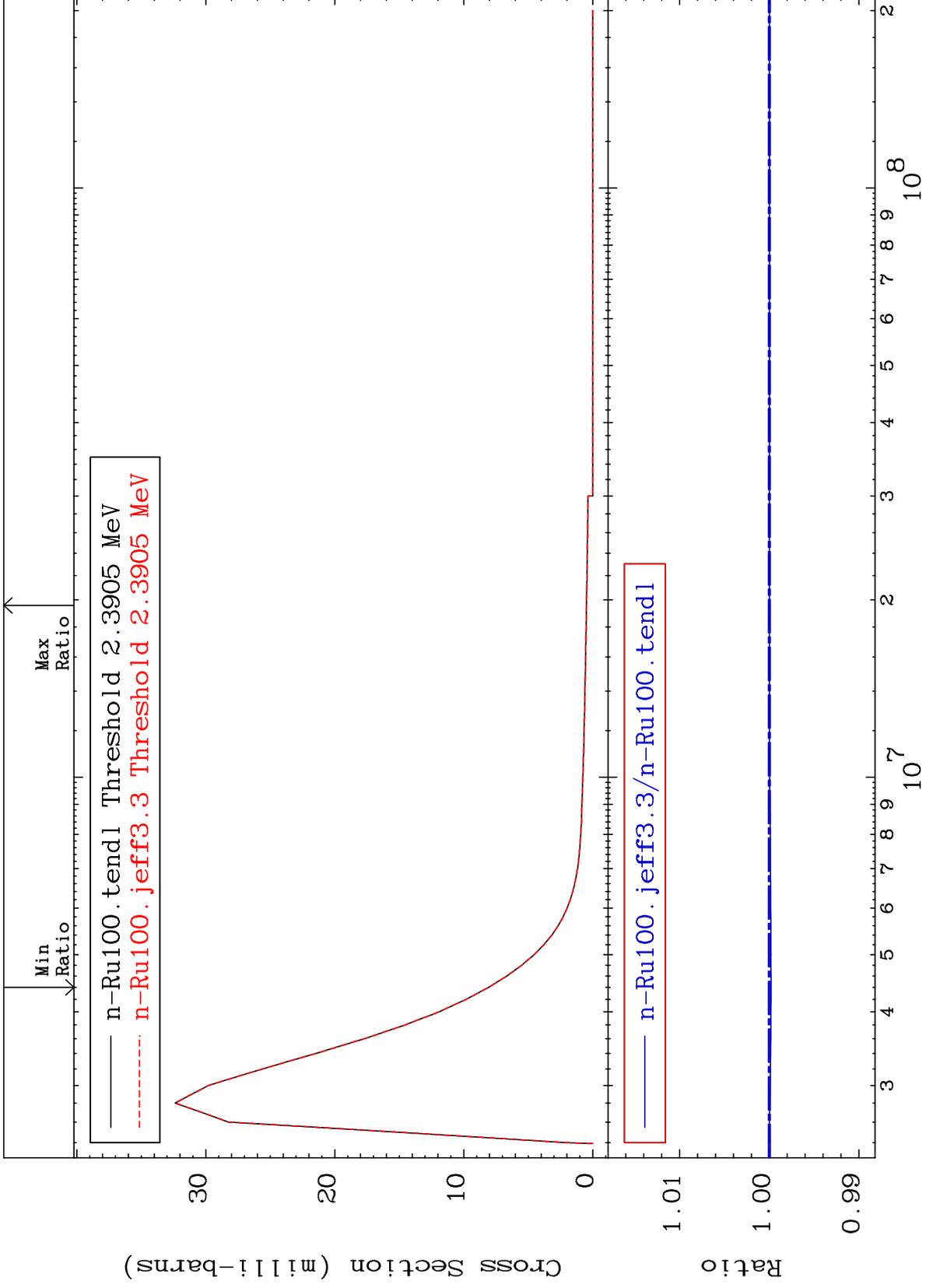
44-Ru-100  
-0.019 To 0.000 %



MAT 4437

MT= 71 (n,n') Level  
Cross Section

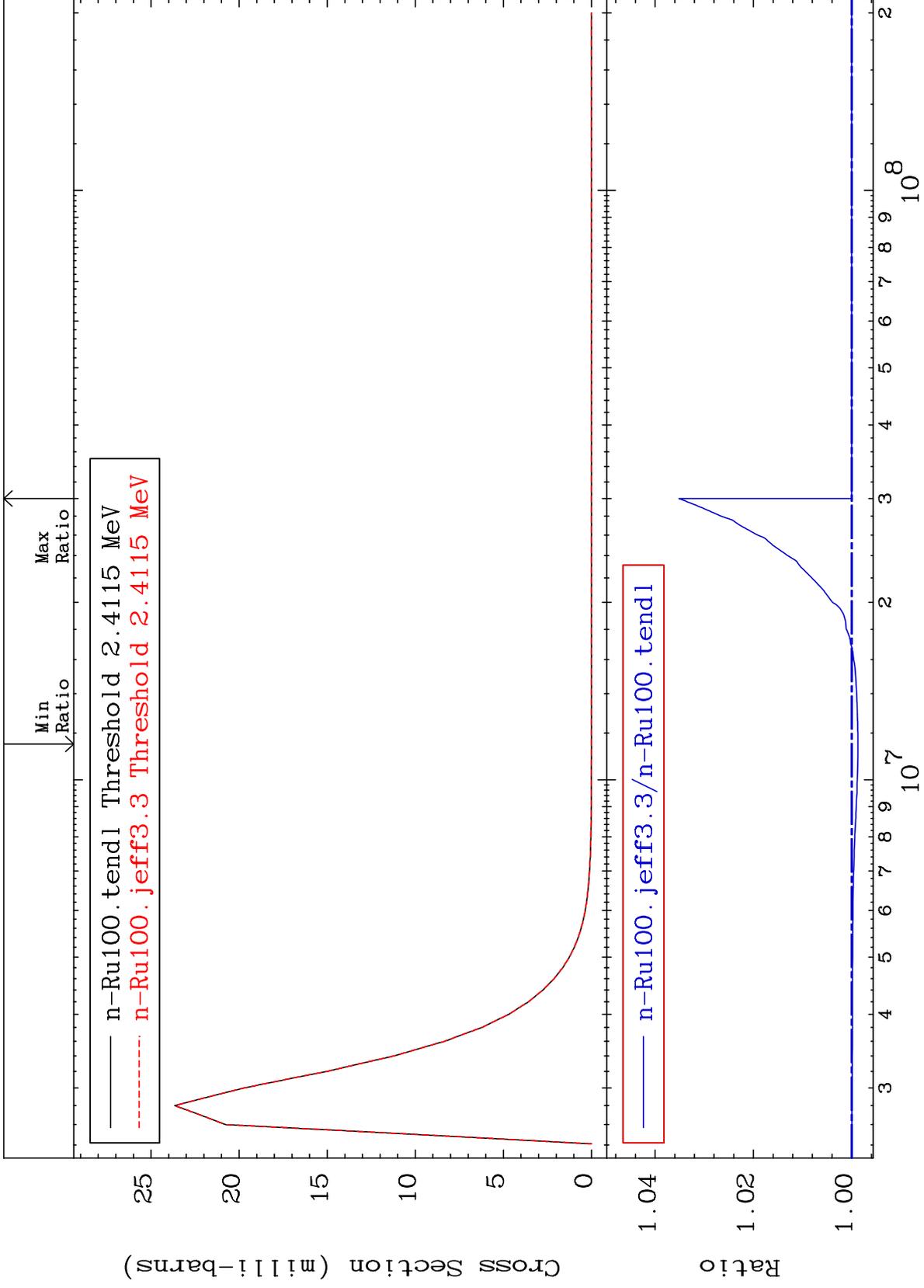
44-Ru-100  
-0.019 To 0.000 %



MAT 4437

MT= 72 (n, n') Level  
Cross Section

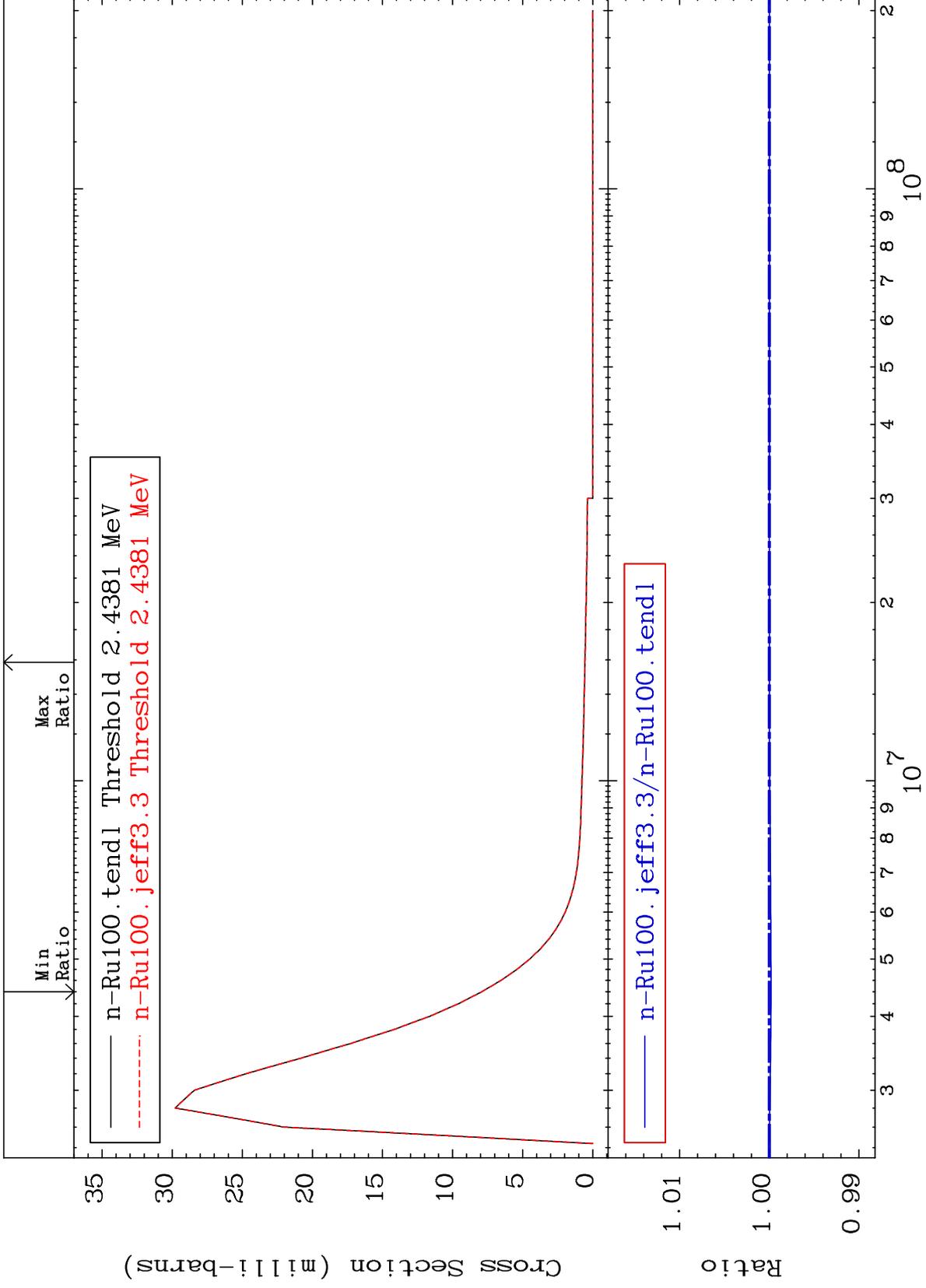
44-Ru-100  
-0.126 To 3.513 %



MAT 4437

MT= 73 (n,n') Level  
Cross Section

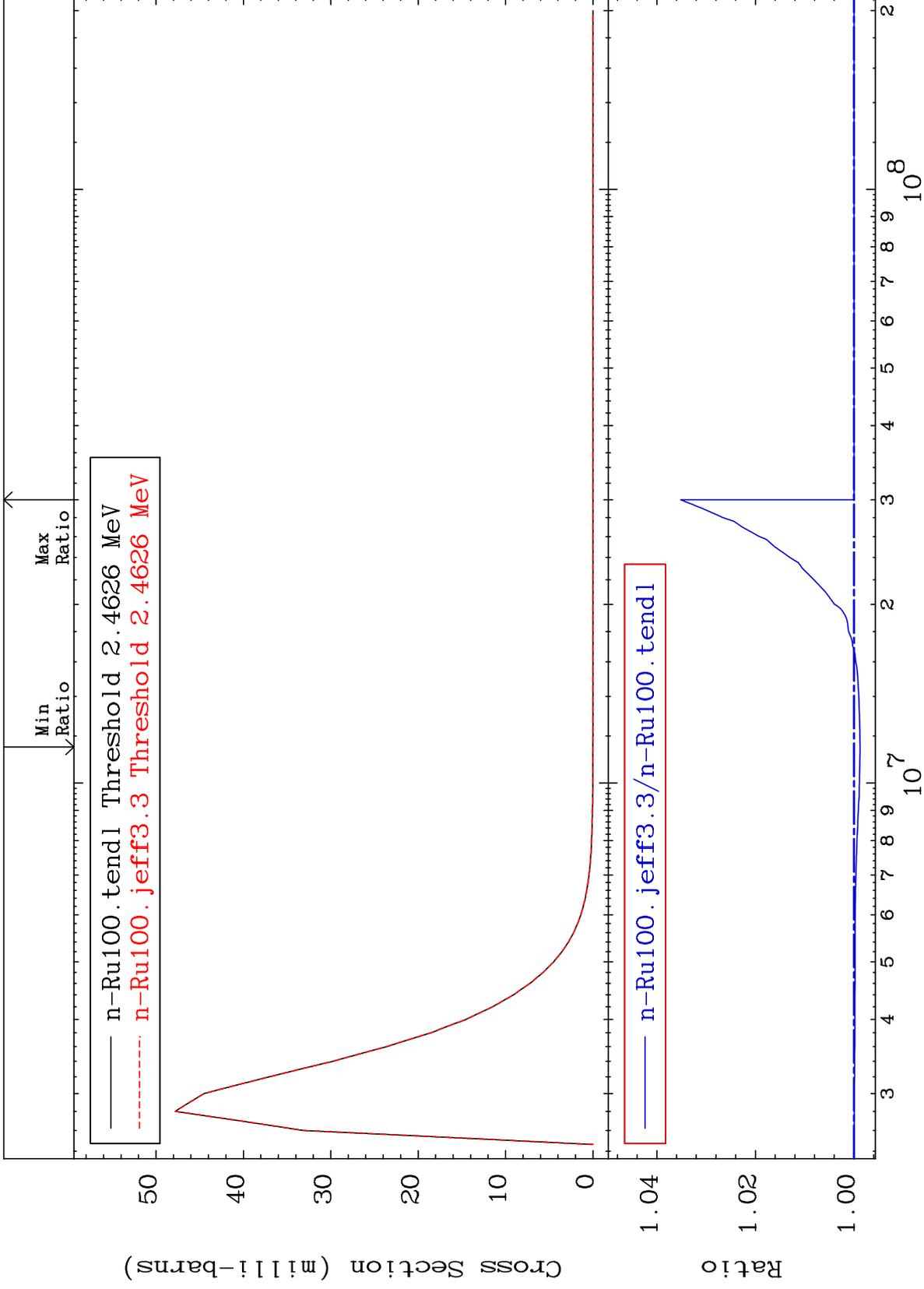
44-Ru-100  
-0.019 To 0.000 %



MAT 4437

MT= 74 (n,n') Level  
Cross Section

44-Ru-100  
-0.123 To 3.515 %



40

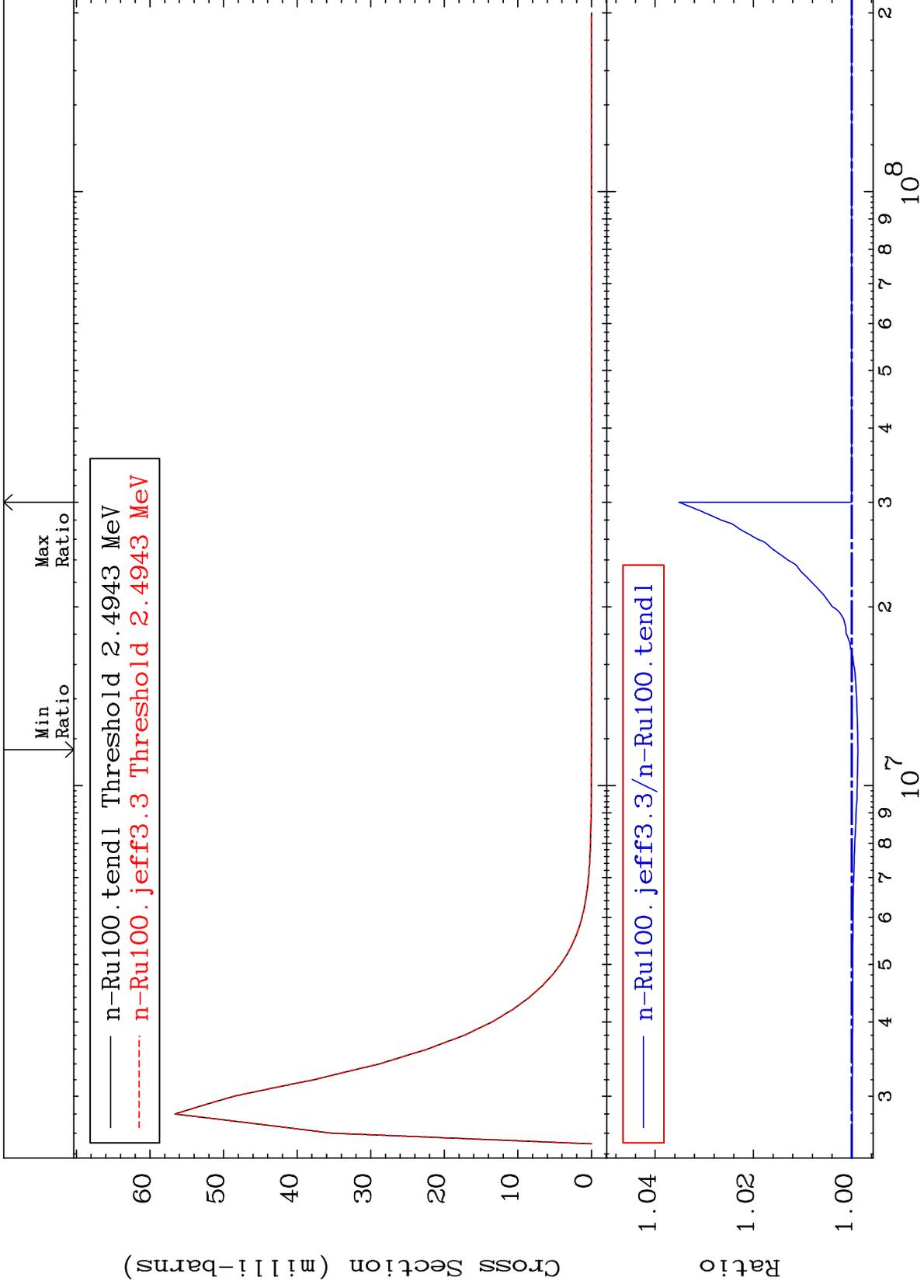
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 75 (n,n') Level  
Cross Section

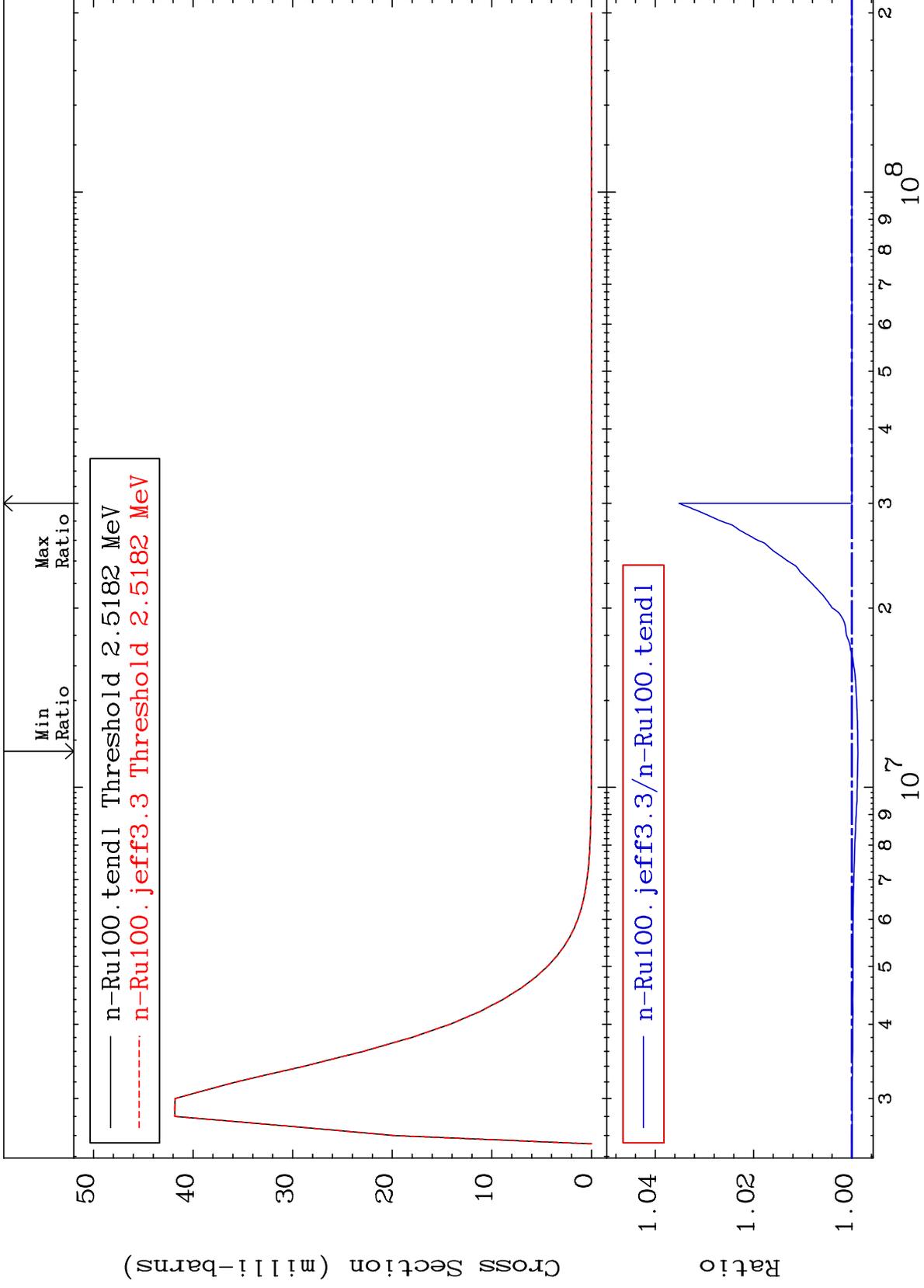
44-Ru-100  
-0.125 To 3.514 %



MAT 4437

MT= 76 (n,n') Level  
Cross Section

44-Ru-100  
-0.123 To 3.514 %



42

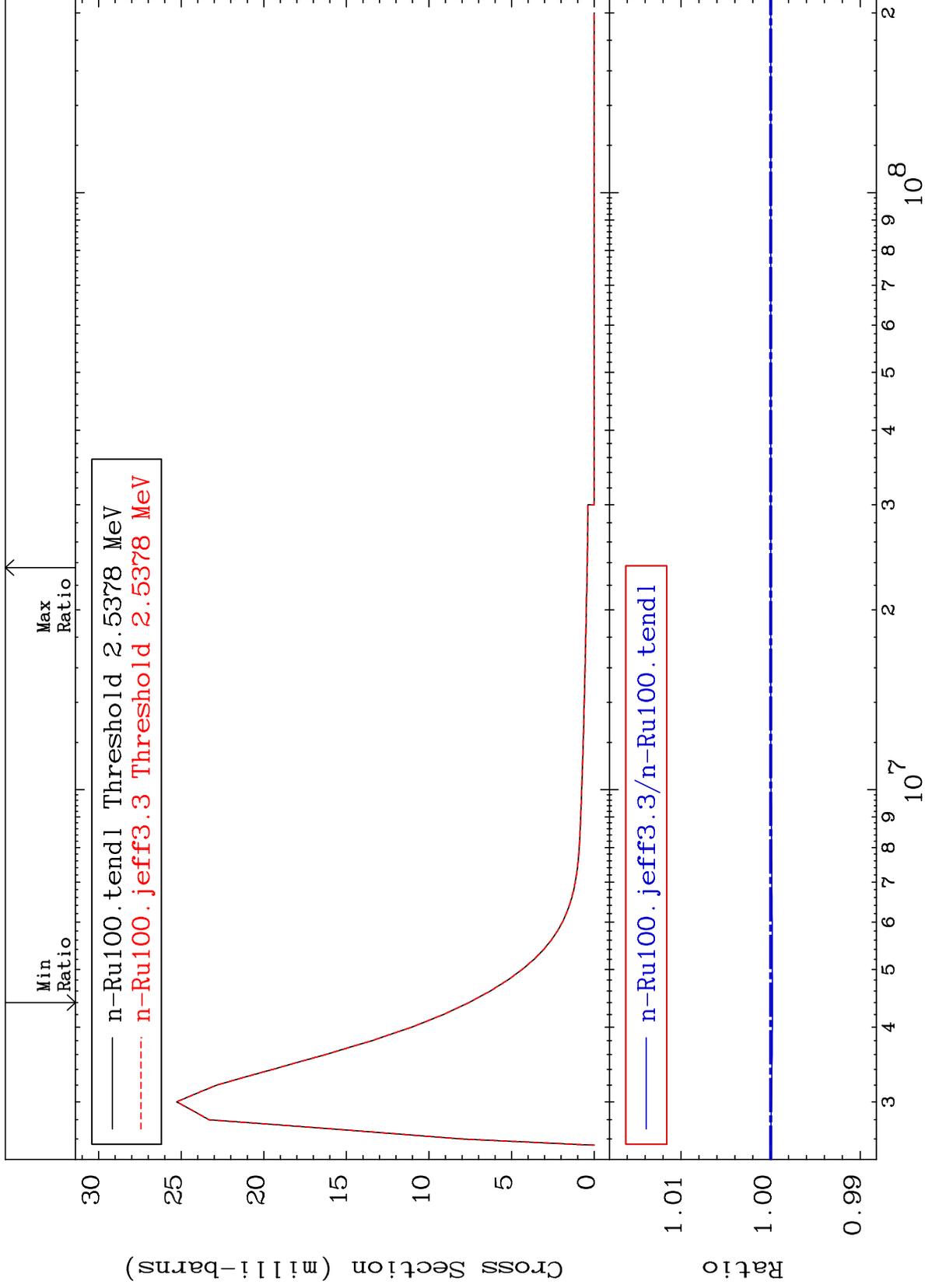
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 77 (n,n') Level  
Cross Section

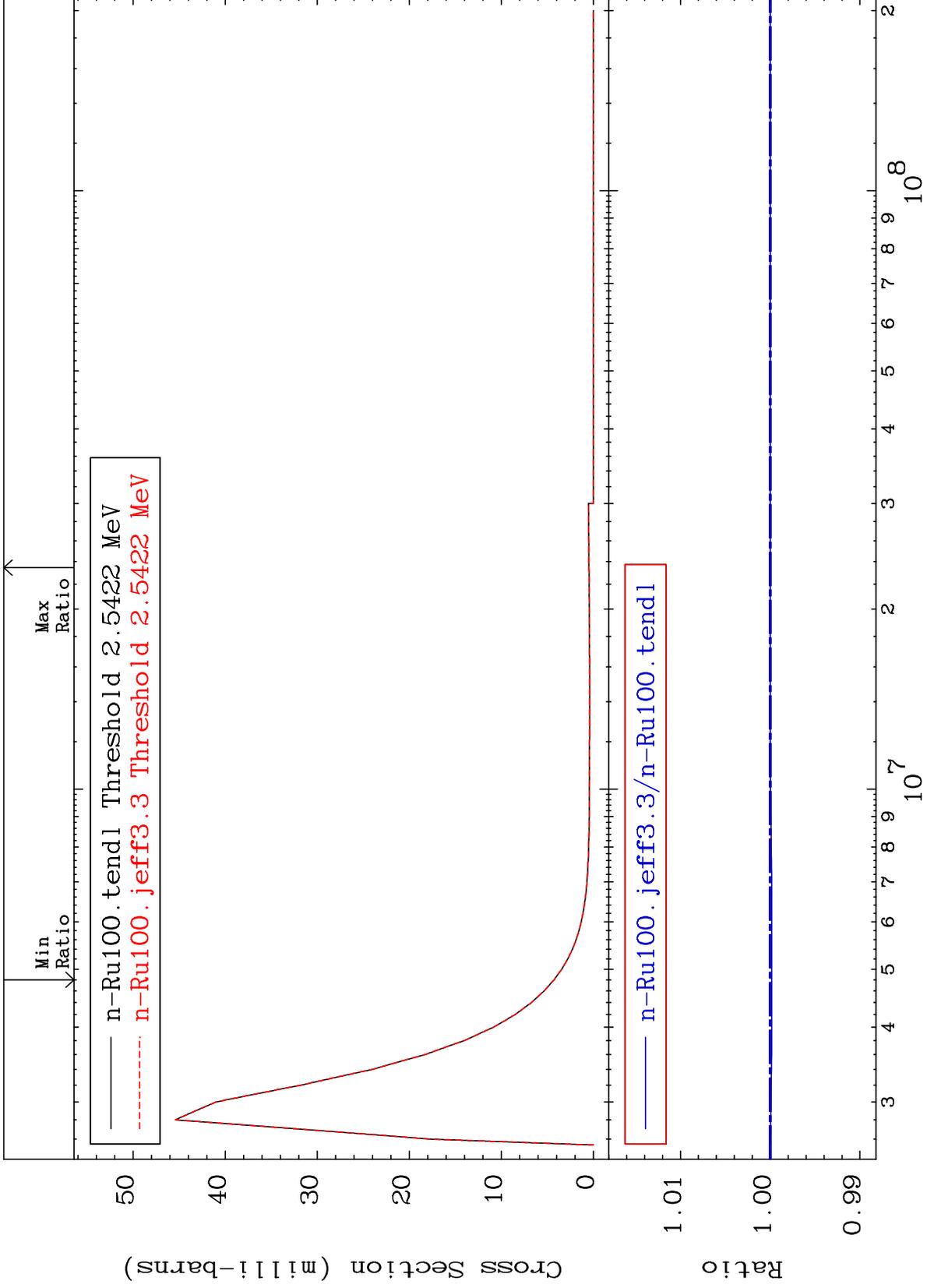
44-Ru-100  
-0.019 To 0.000 %



MAT 4437

MT= 78 (n,n') Level  
Cross Section

44-Ru-100  
-0.017 To 0.000 %



44

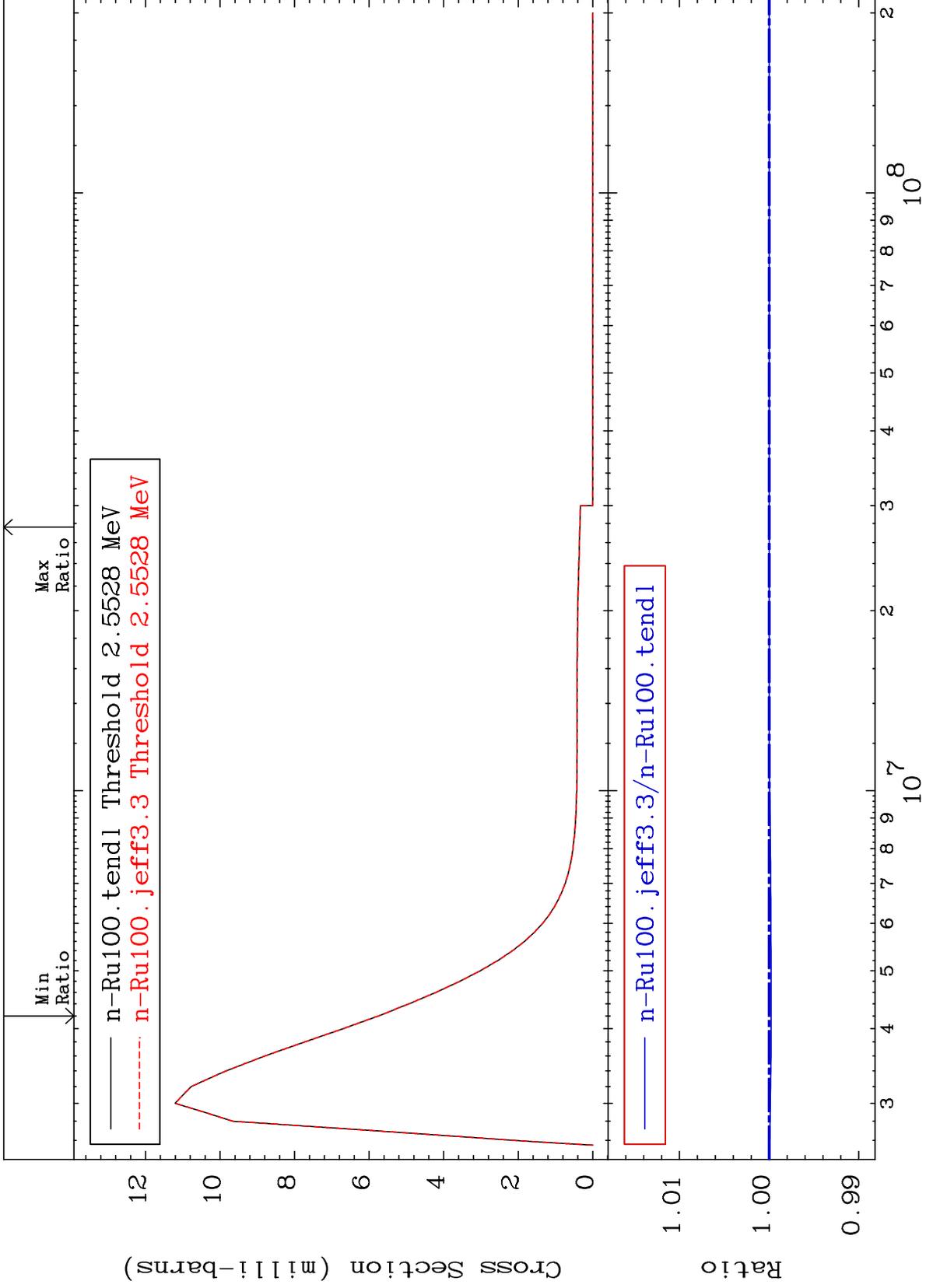
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 79 (n,n') Level  
Cross Section

44-Ru-100  
-0.022 To 0.000 %



45

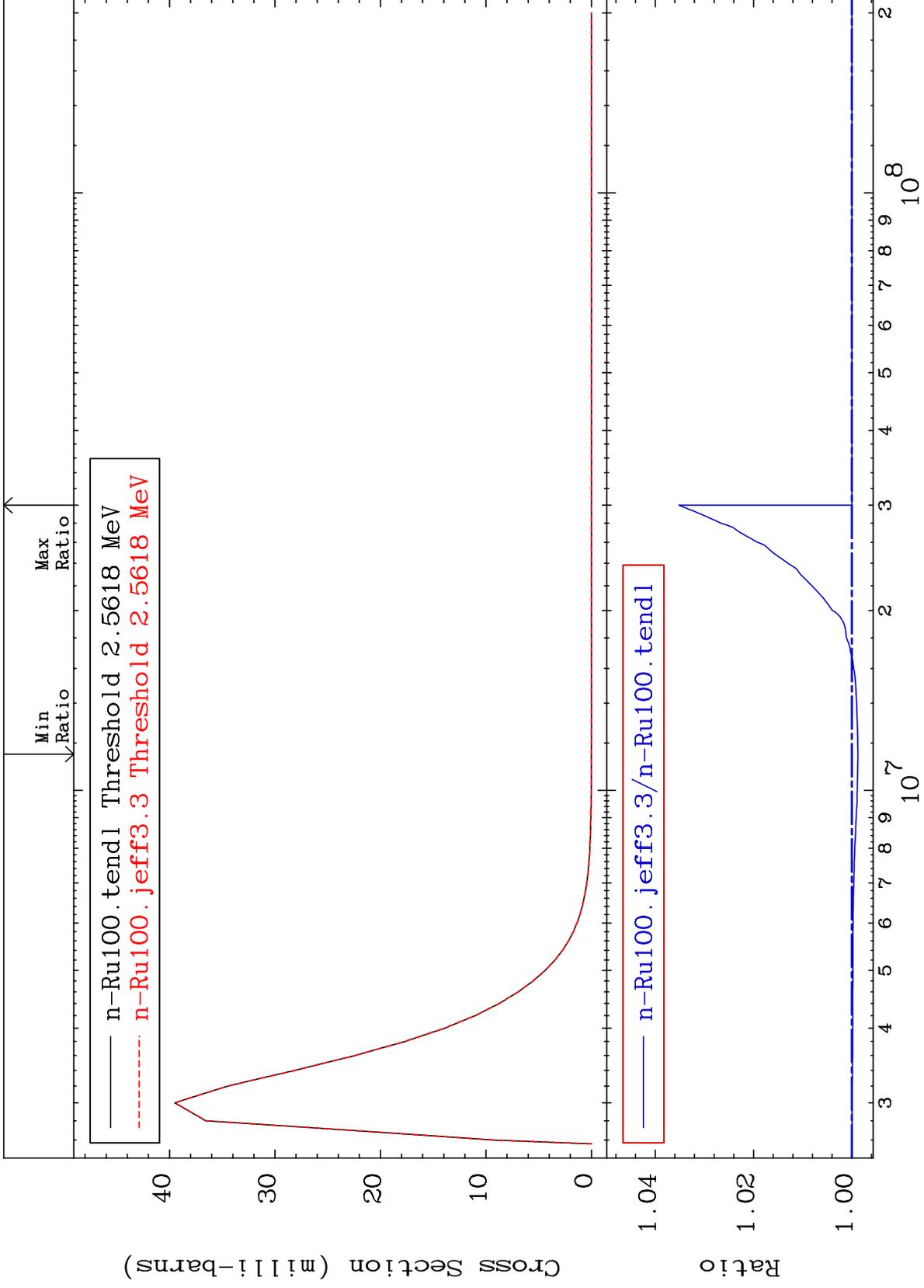
Incident Energy (eV)

44-Ru-100

MAT 4437

MT= 80 (n,n') Level  
Cross Section

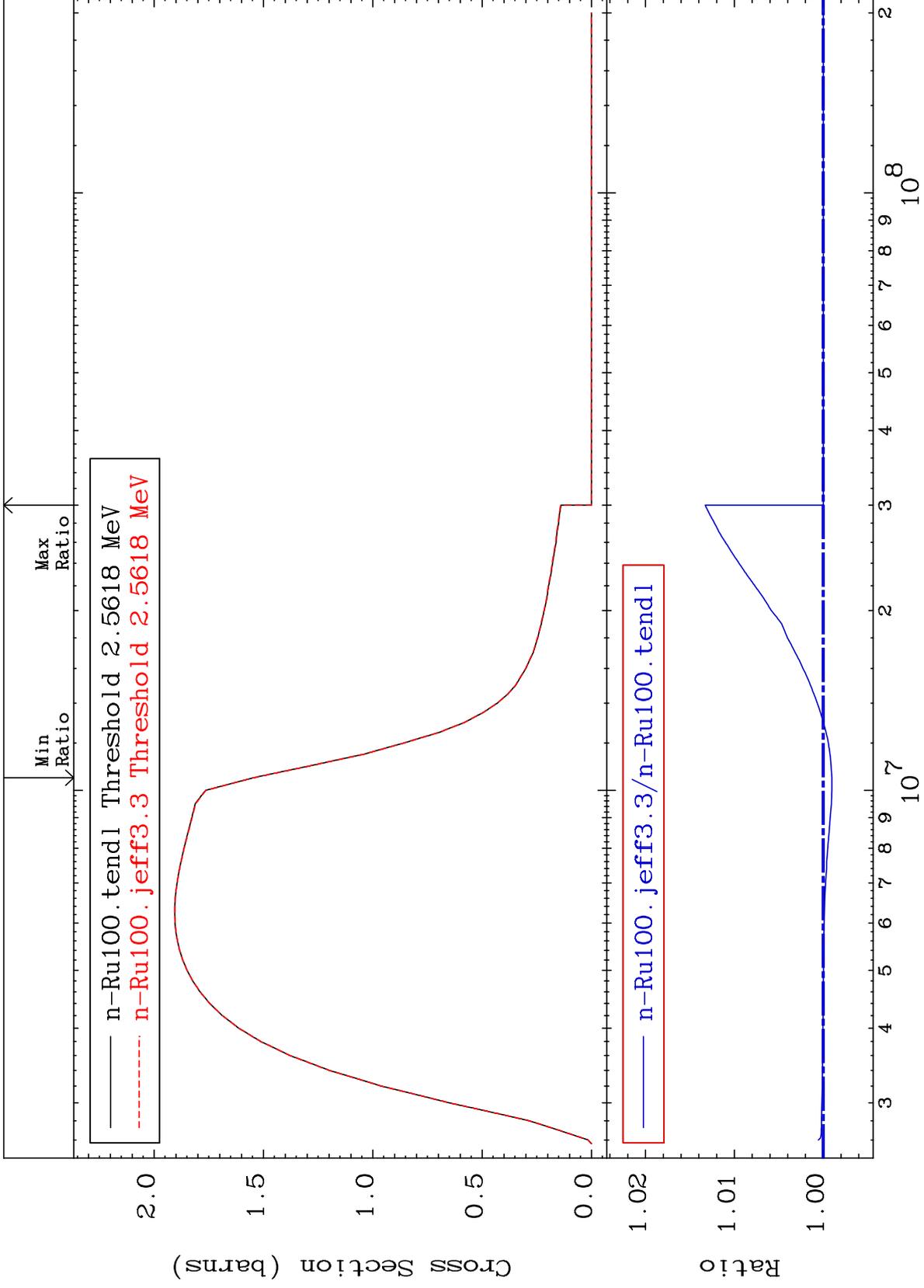
44-Ru-100  
-0.123 To 3.514 %



MAT 4437

(n, n') Continuum  
Cross Section

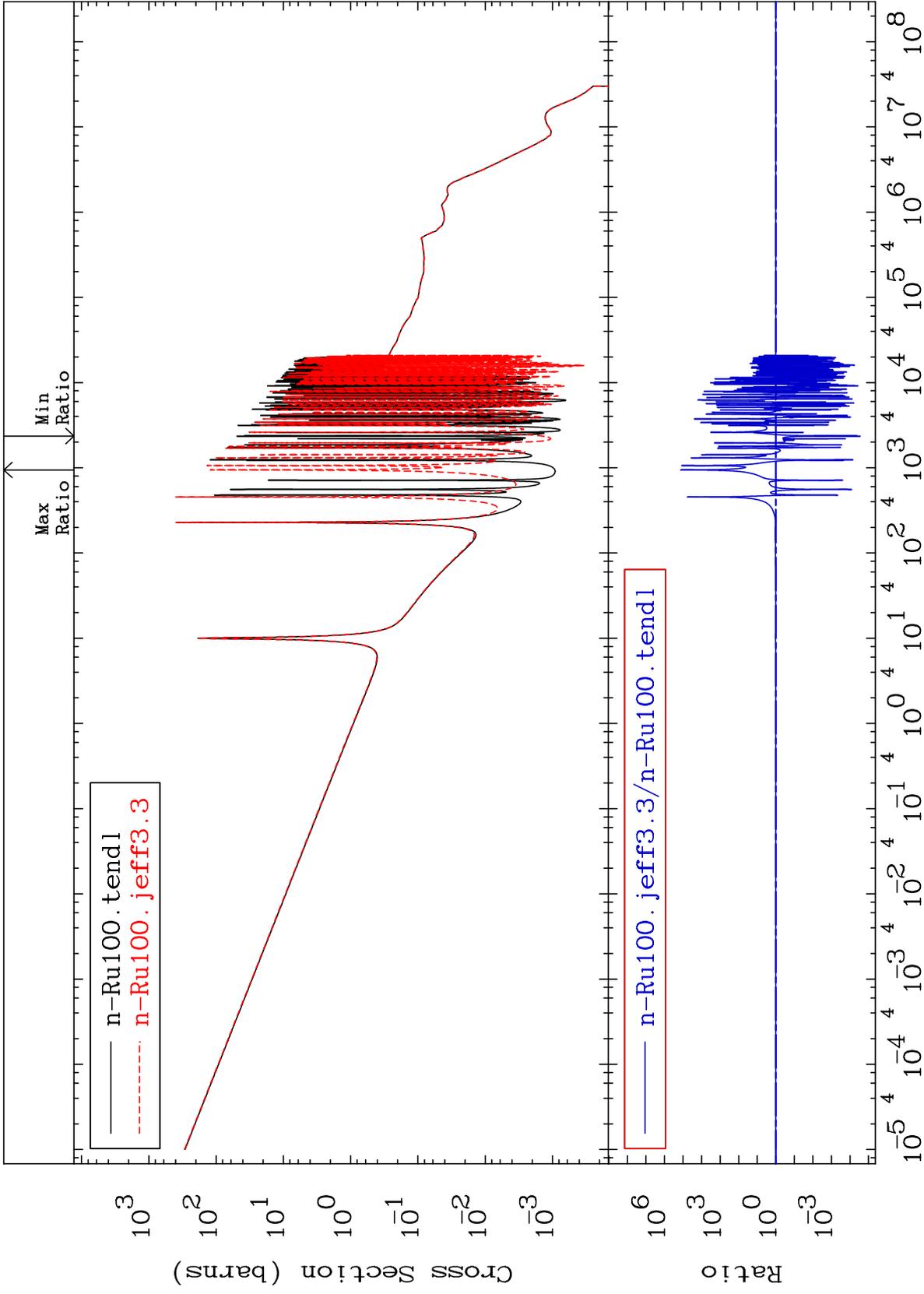
44-Ru-100  
-0.096 To 1.329 %



MAT 4437

(n,  $\gamma$ )  
Cross Section

44-Ru-100  
-100.0 To 9999. %



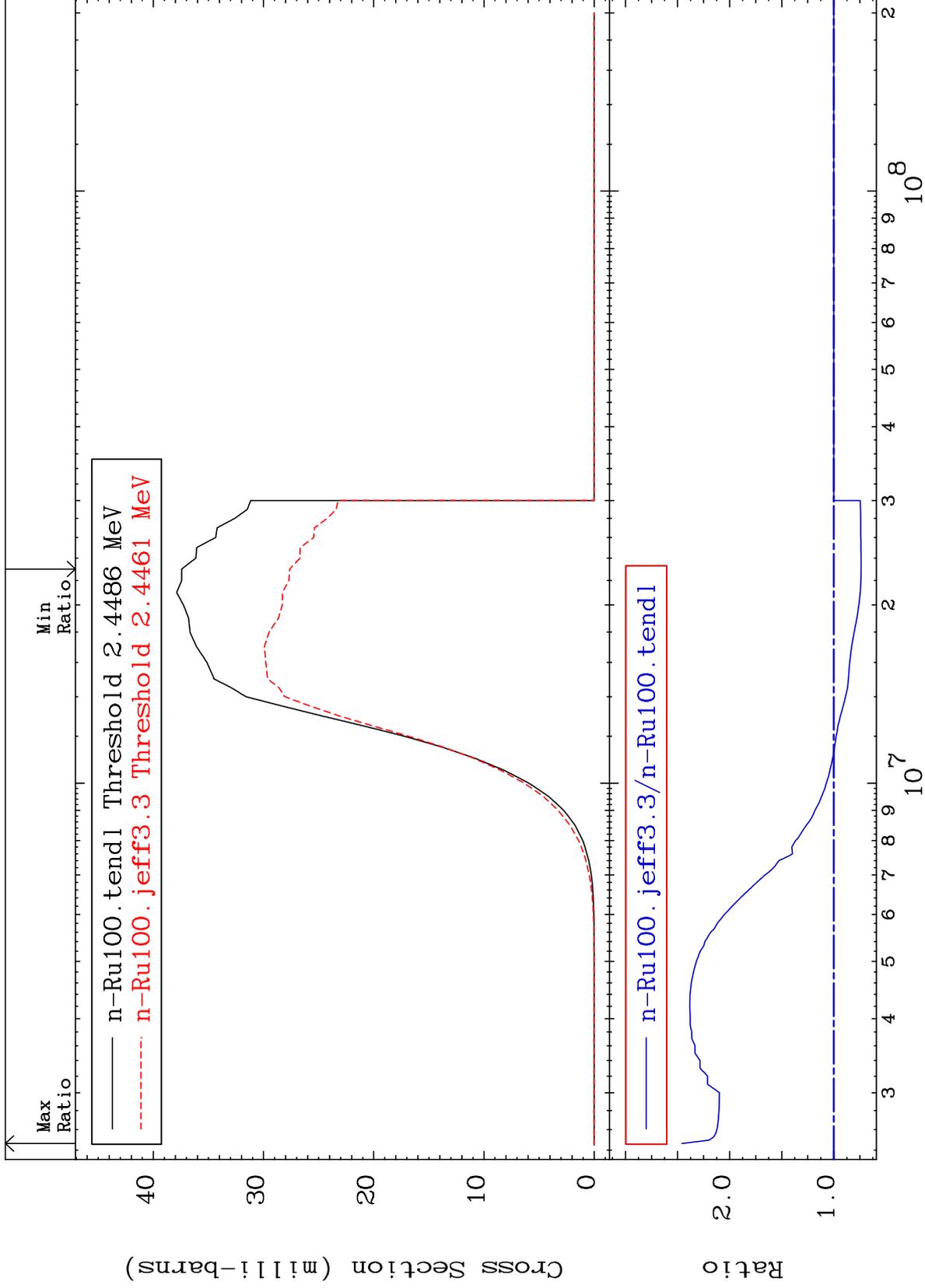
MAT 4437

(n, p)

44-Ru-100

Cross Section

-26.21 To 145.8 %



MAT 4437

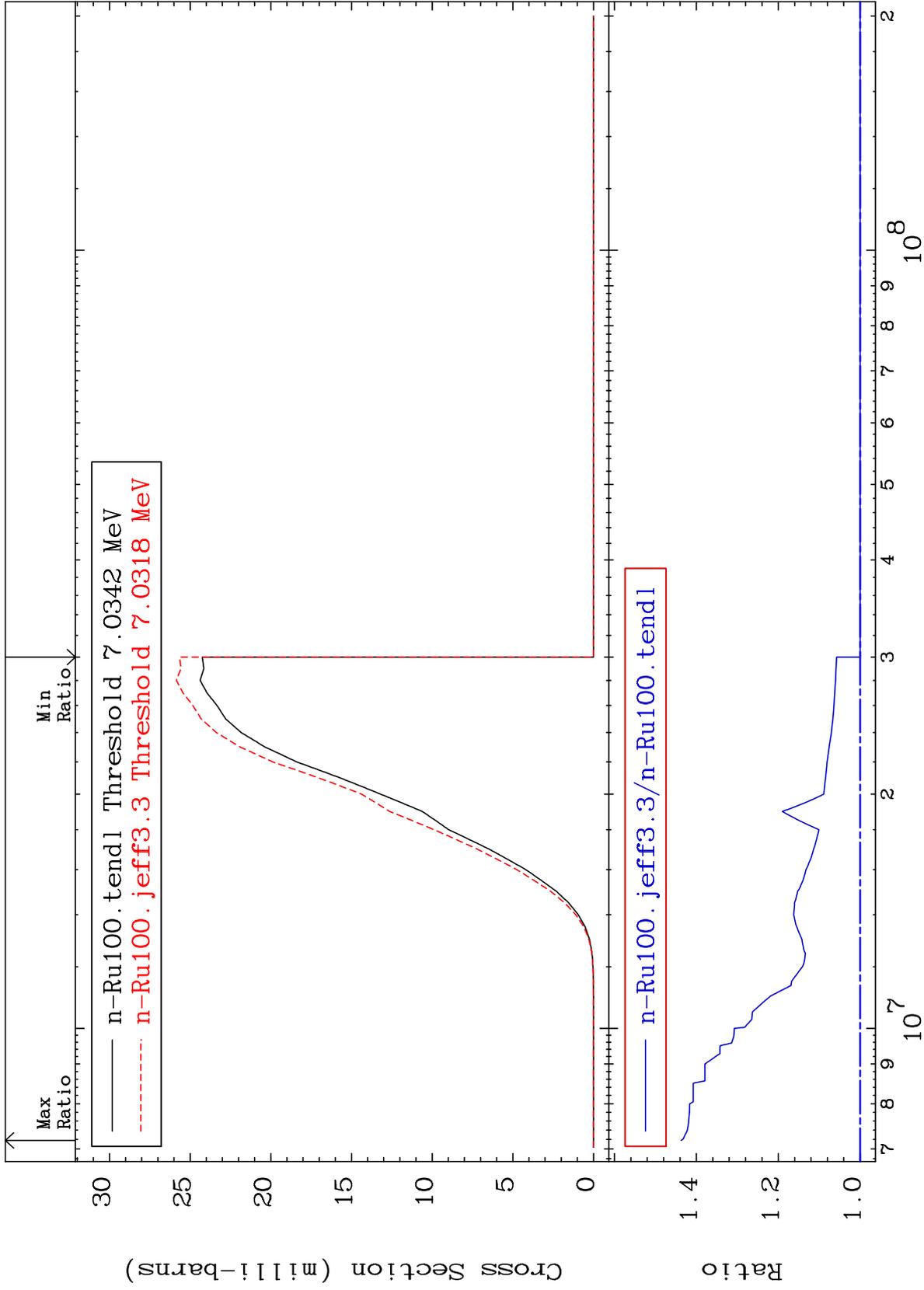
(n, d)

44-Ru-100

Cross Section

0.000

To 43.72 %

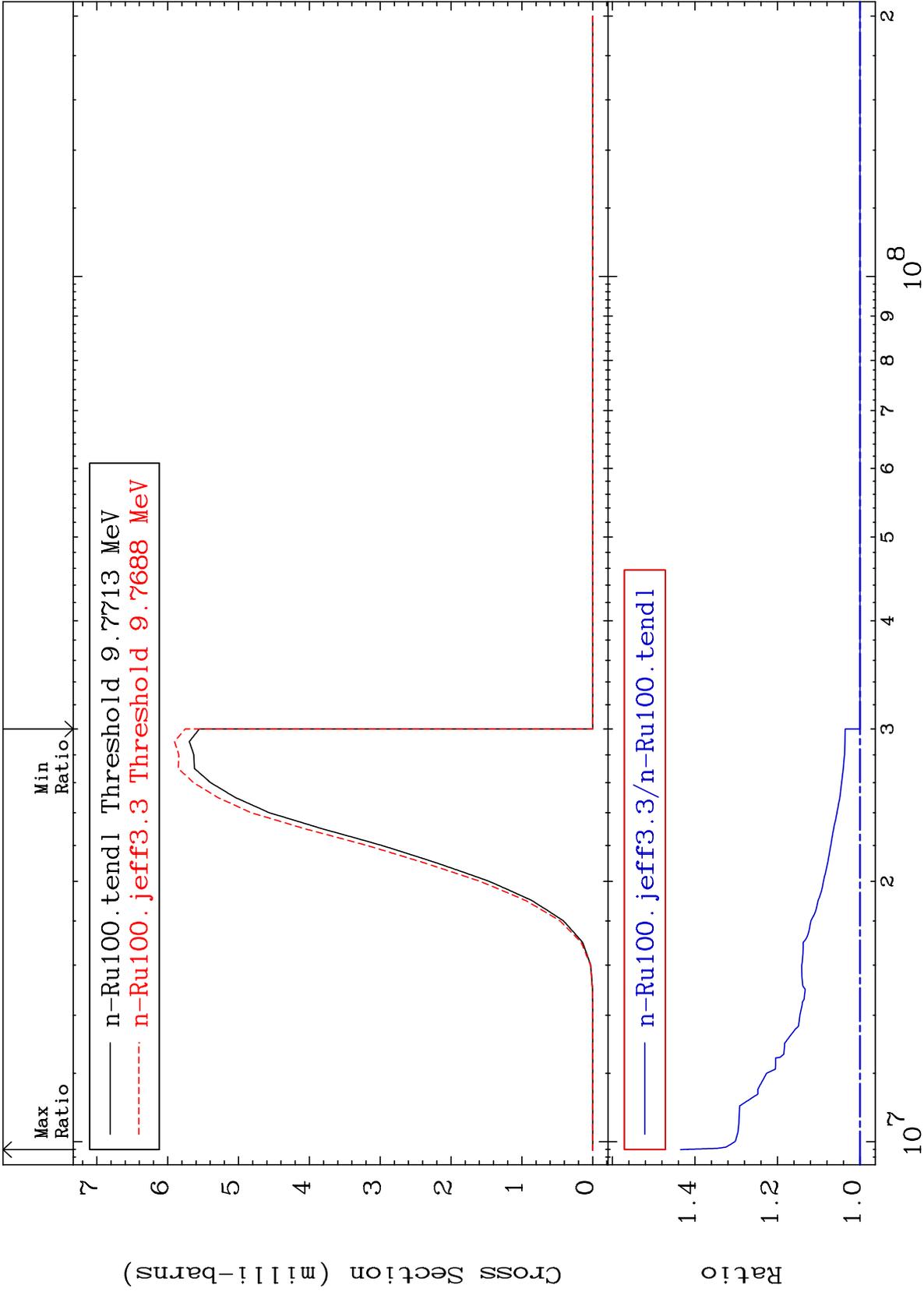


MAT 4437

44-Ru-100

0.000 To 43.49 %

(n, t)  
Cross Section



51

Incident Energy (eV)

44-Ru-100

MAT 4437

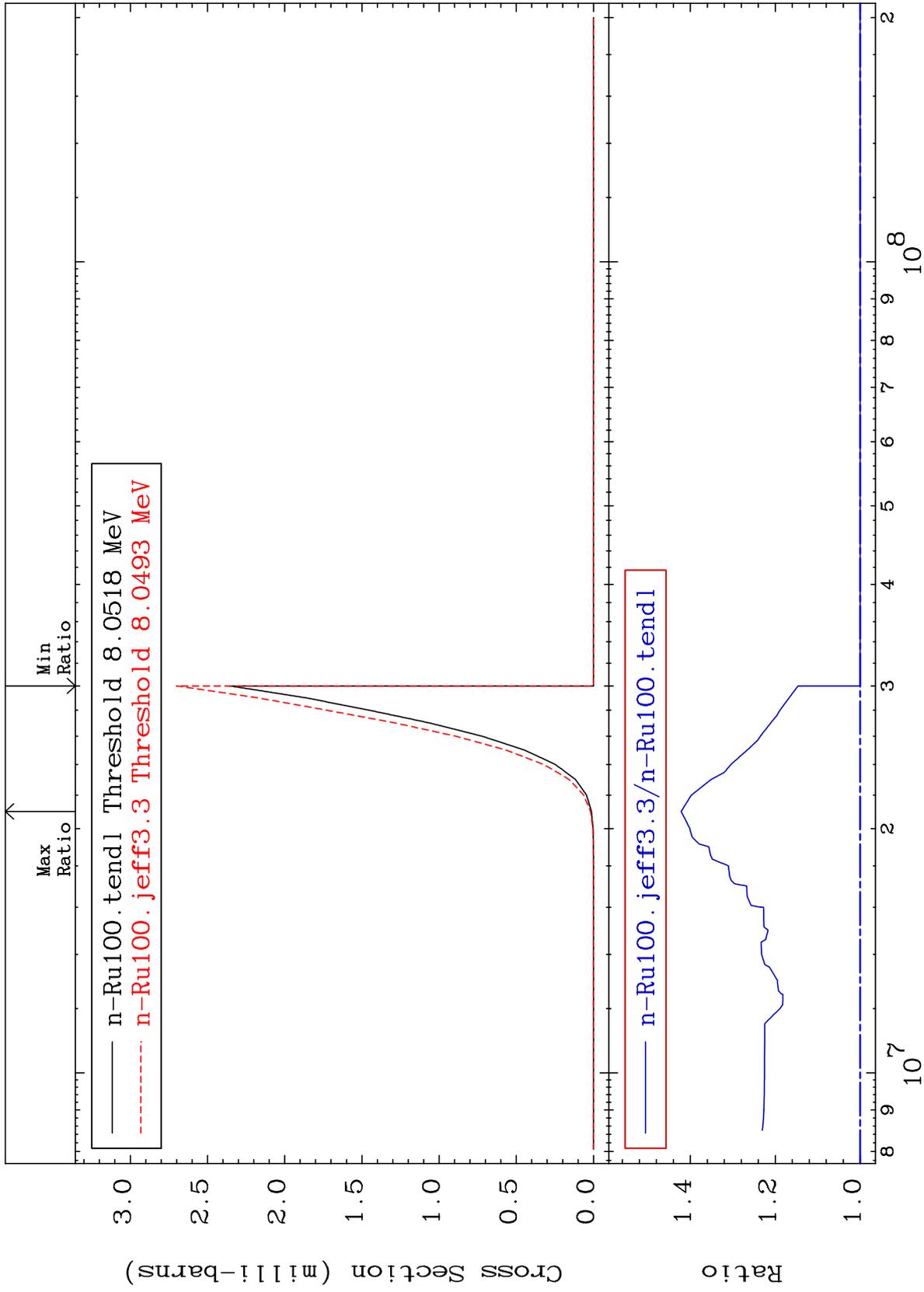
(n, He-3)

44-Ru-100

Cross Section

0.000

To 42.15 %



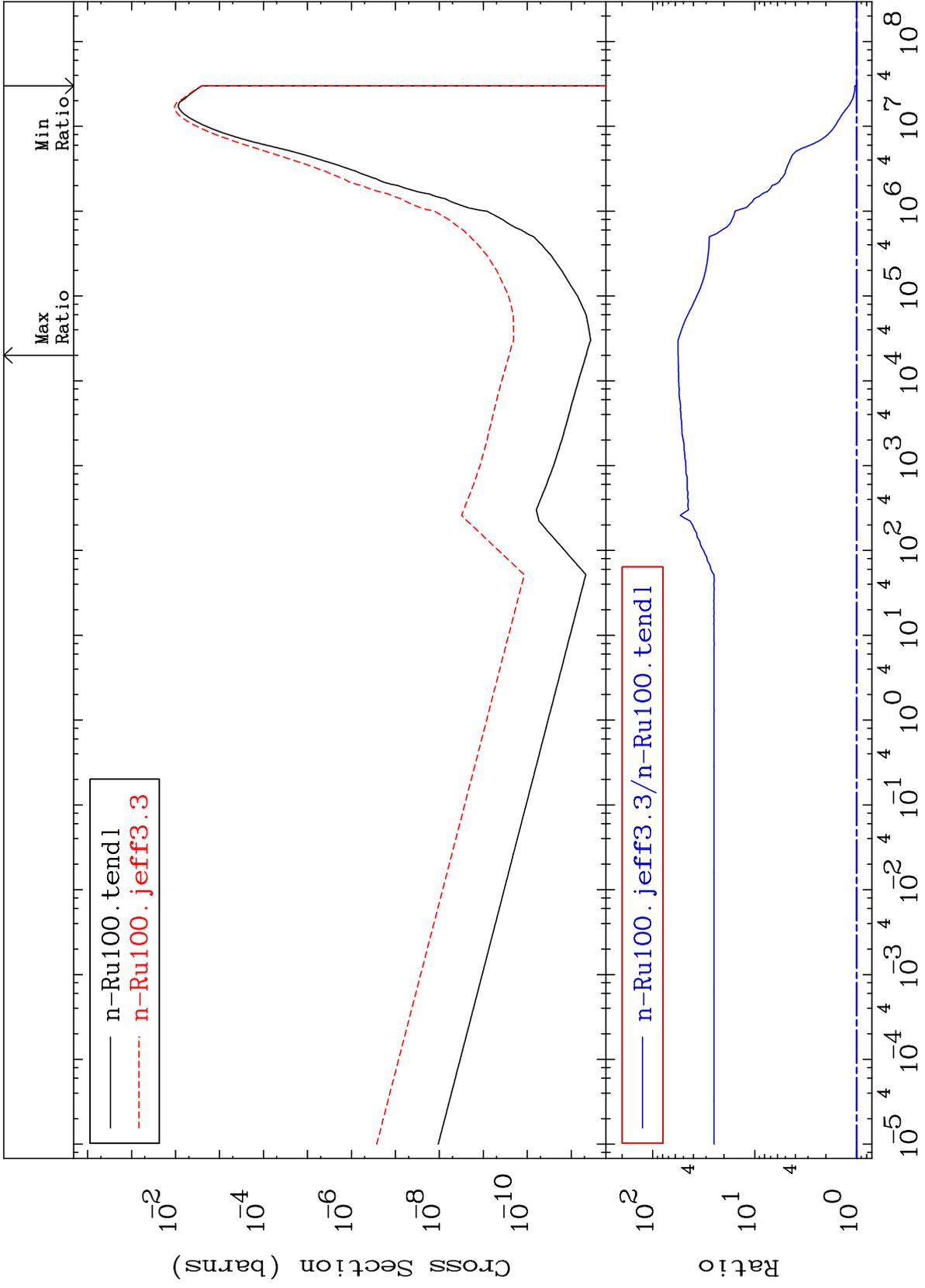
52

Incident Energy (eV)

44-Ru-100

MAT 4437

(n,  $\alpha$ )  
Cross Section  
44-Ru-100  
To 5545. %

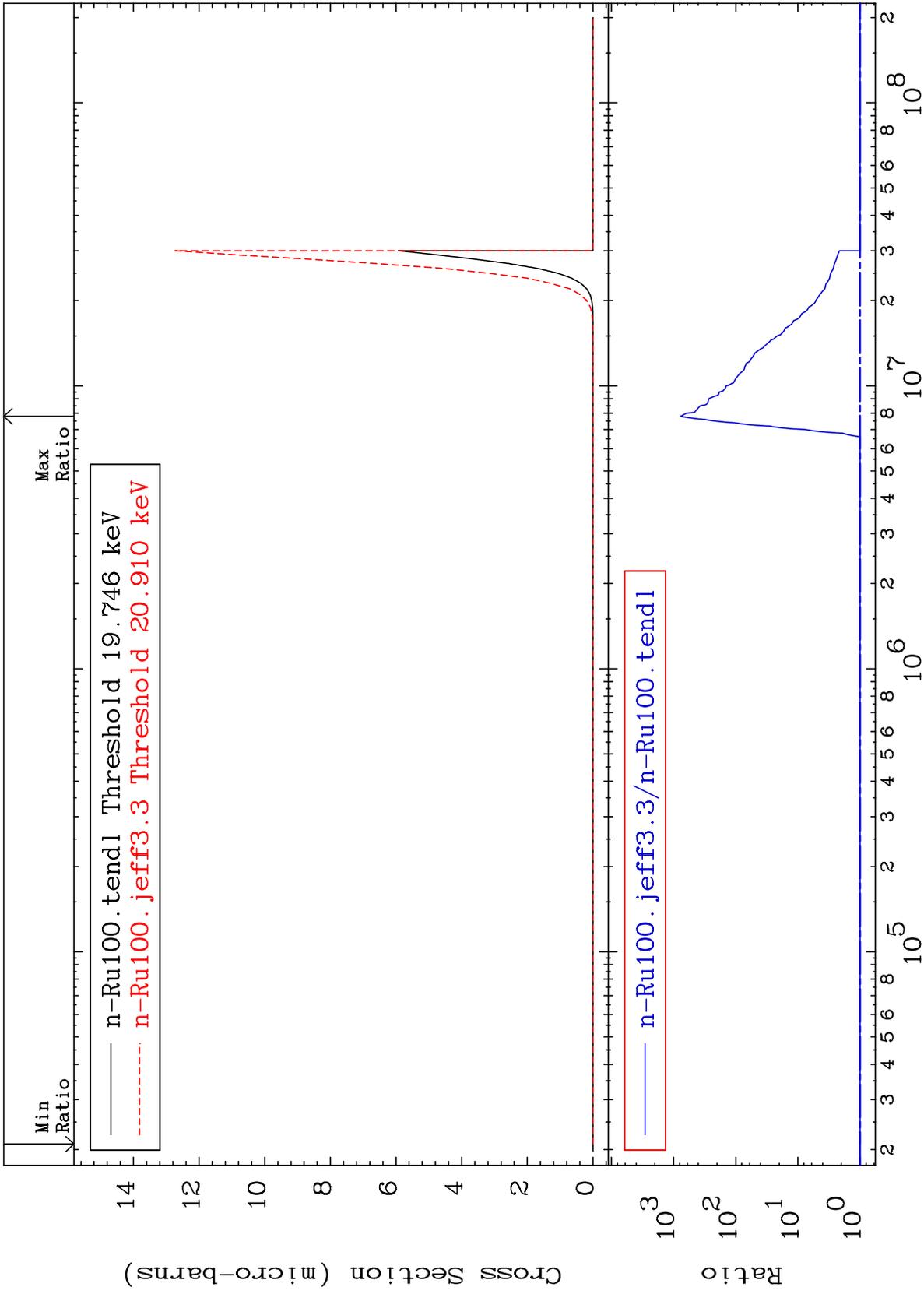


MAT 4437

(n,2α)

44-Ru-100  
To 9999. %

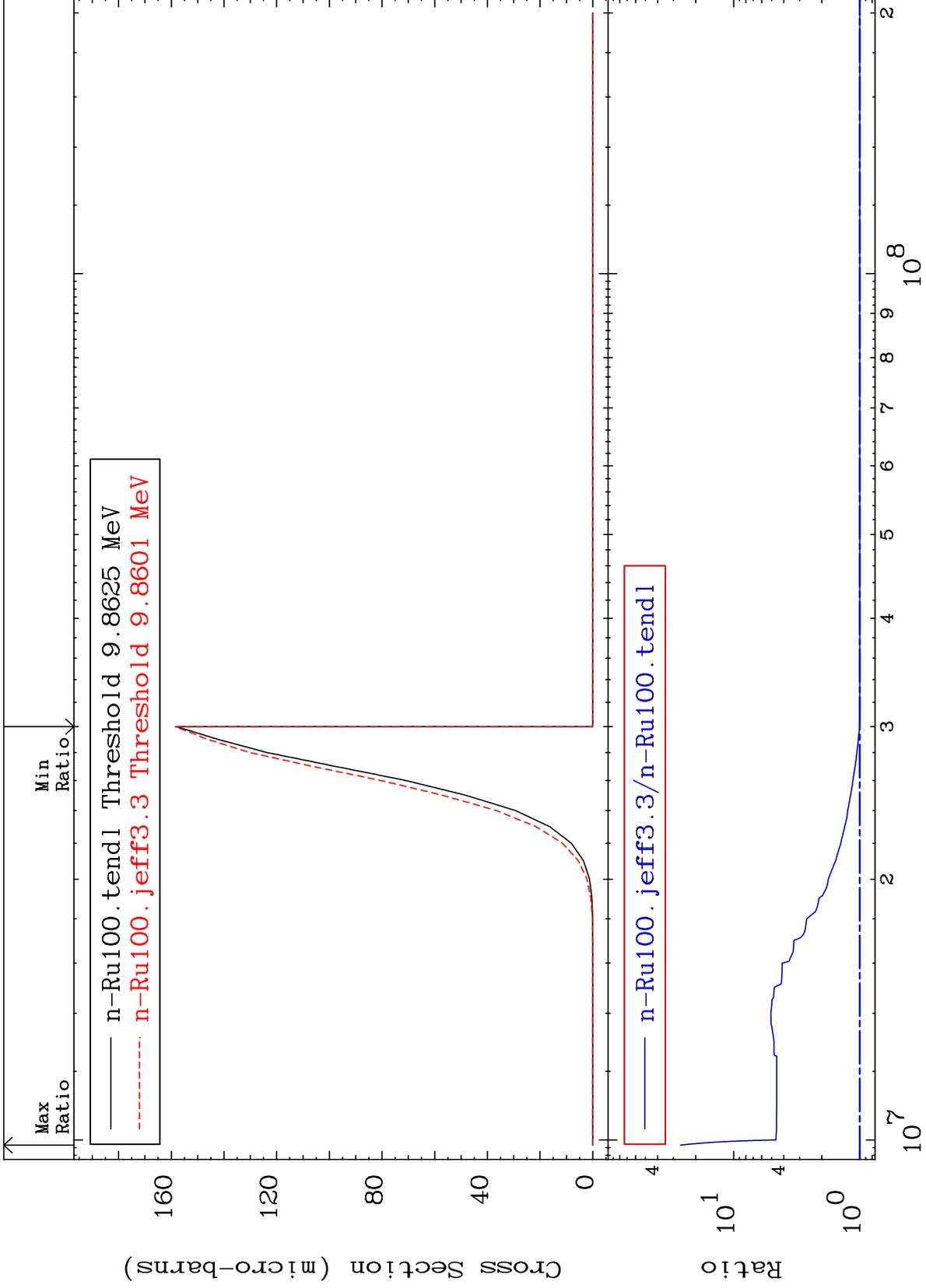
Cross Section



MAT 4437

(n,2p)

Cross Section  
0.000 To 2538. %  
44-Ru-100



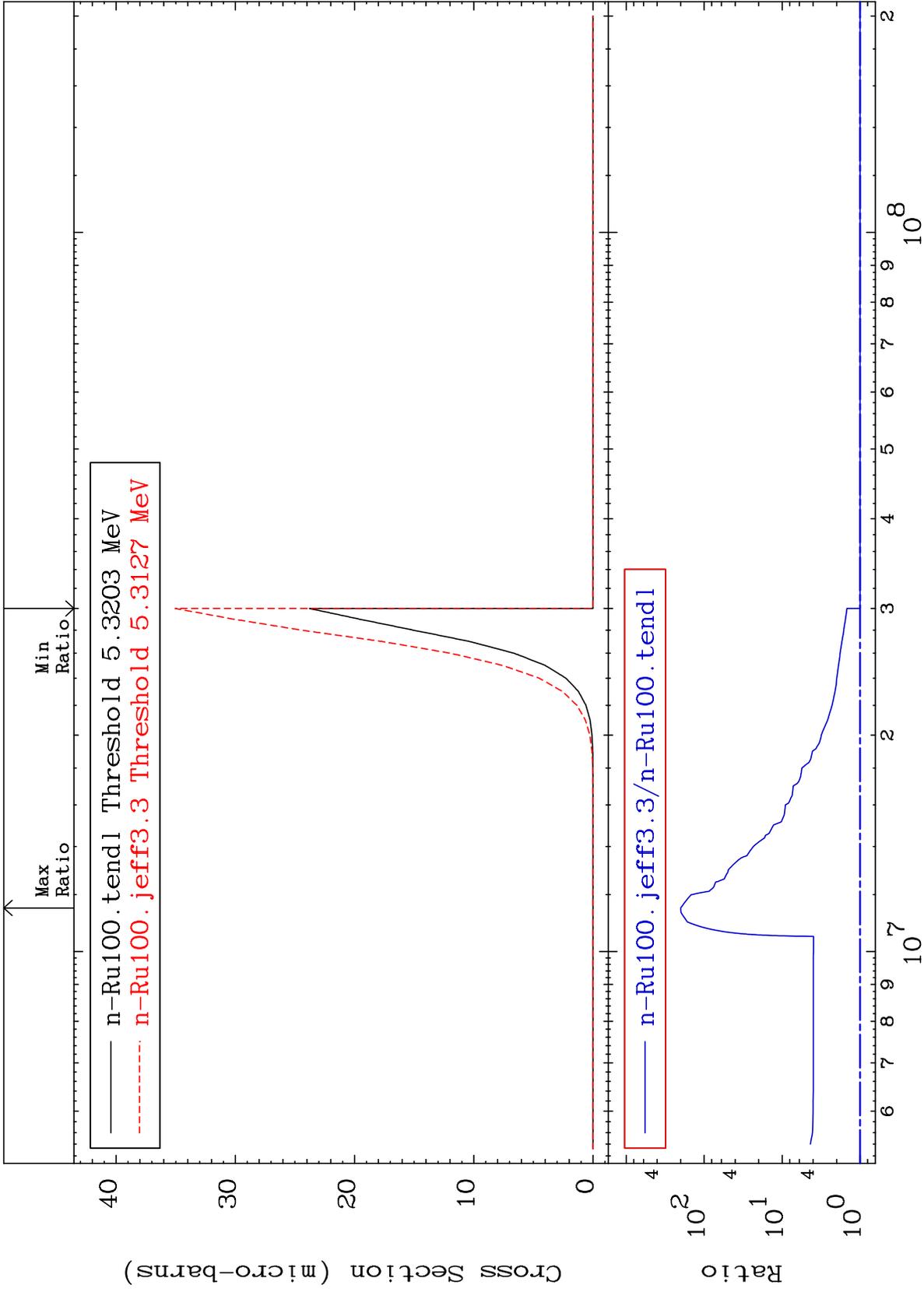
55

44-Ru-100

MAT 4437

(n,p)  $\alpha$   
Cross Section

44-Ru-100  
To 9999. %



MAT 4437

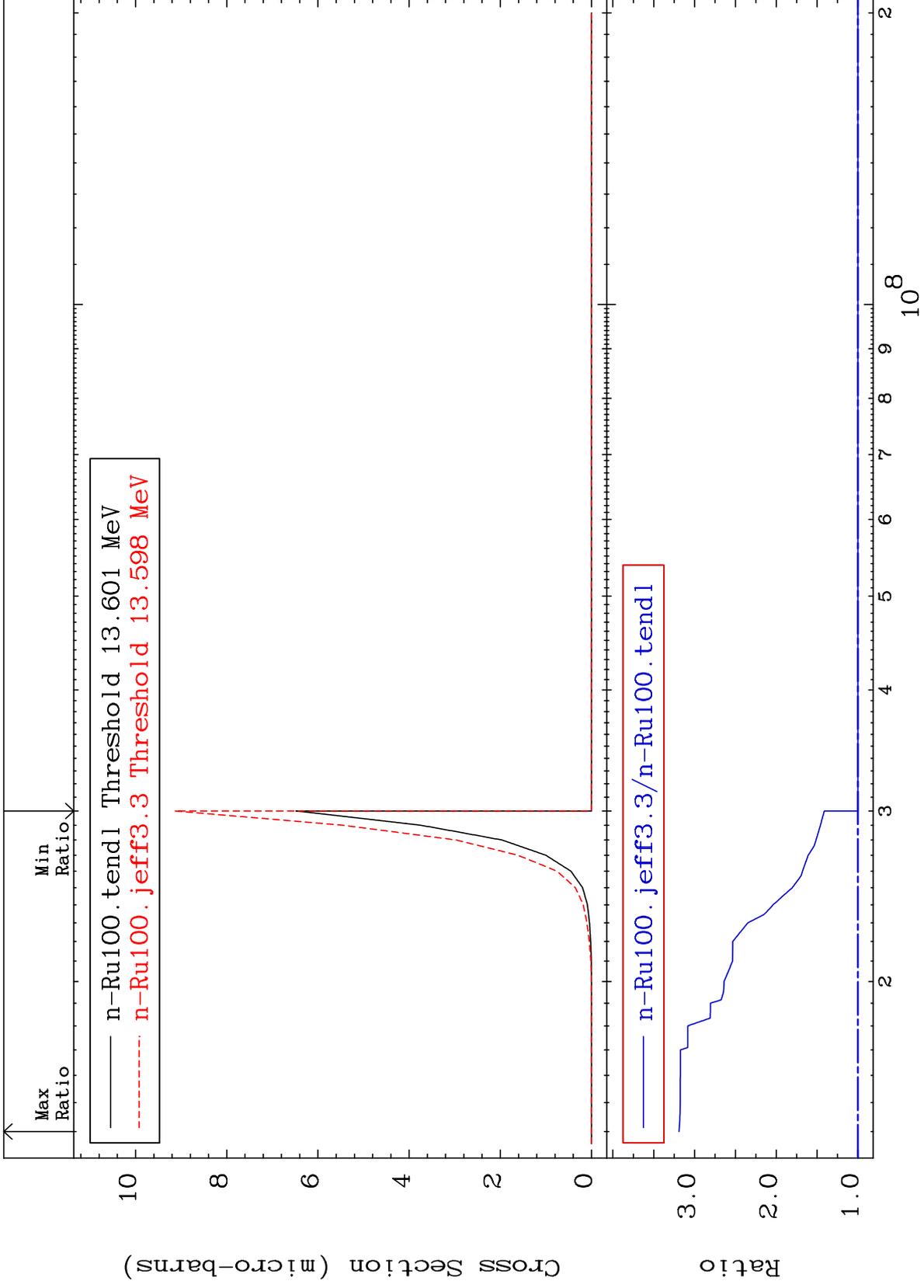
(n, p) d

44-Ru-100

Cross Section

0.000

To 219.0 %



MAT 4437

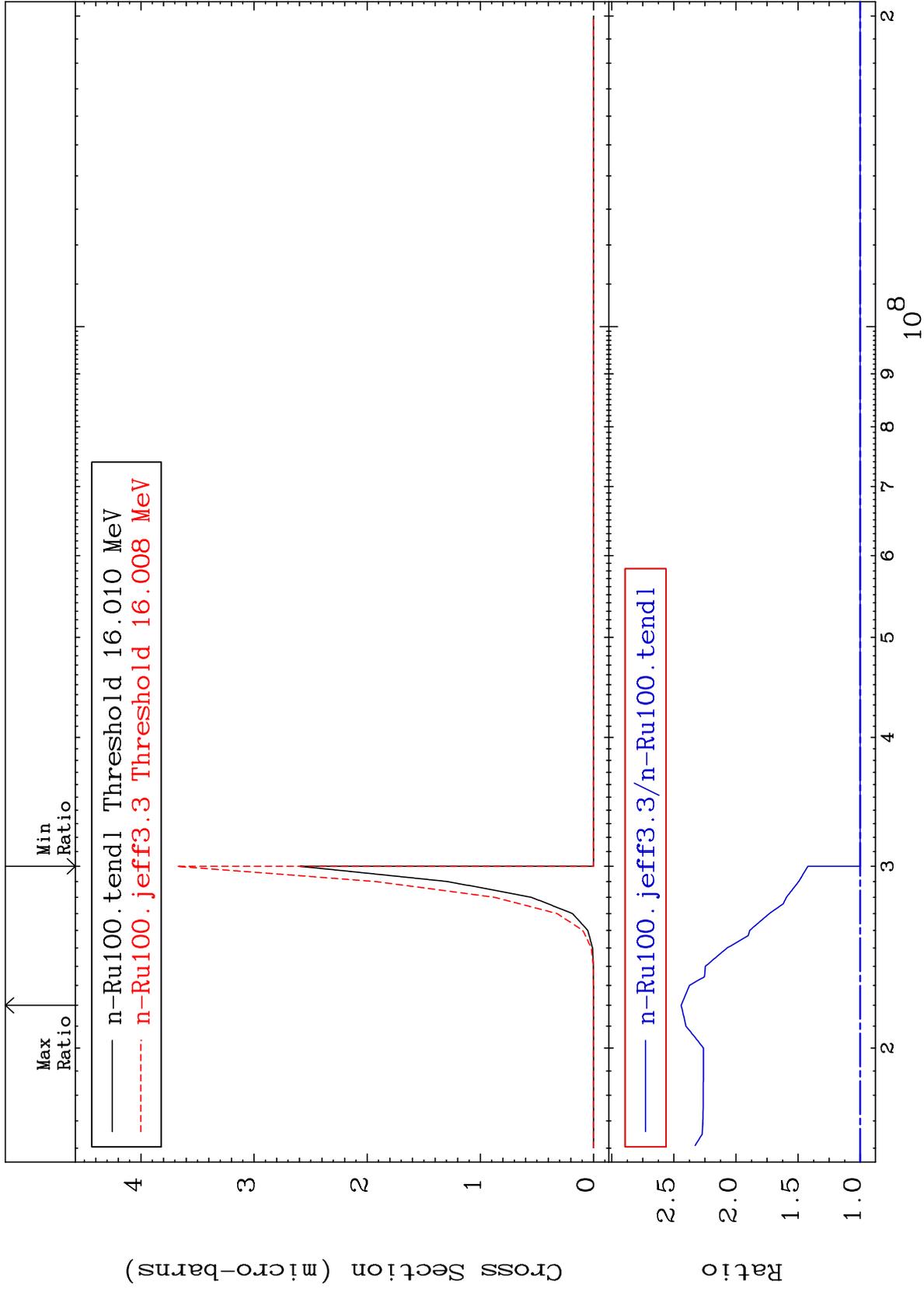
(n,p) t

44-Ru-100

Cross Section

0.000

To 144.1 %



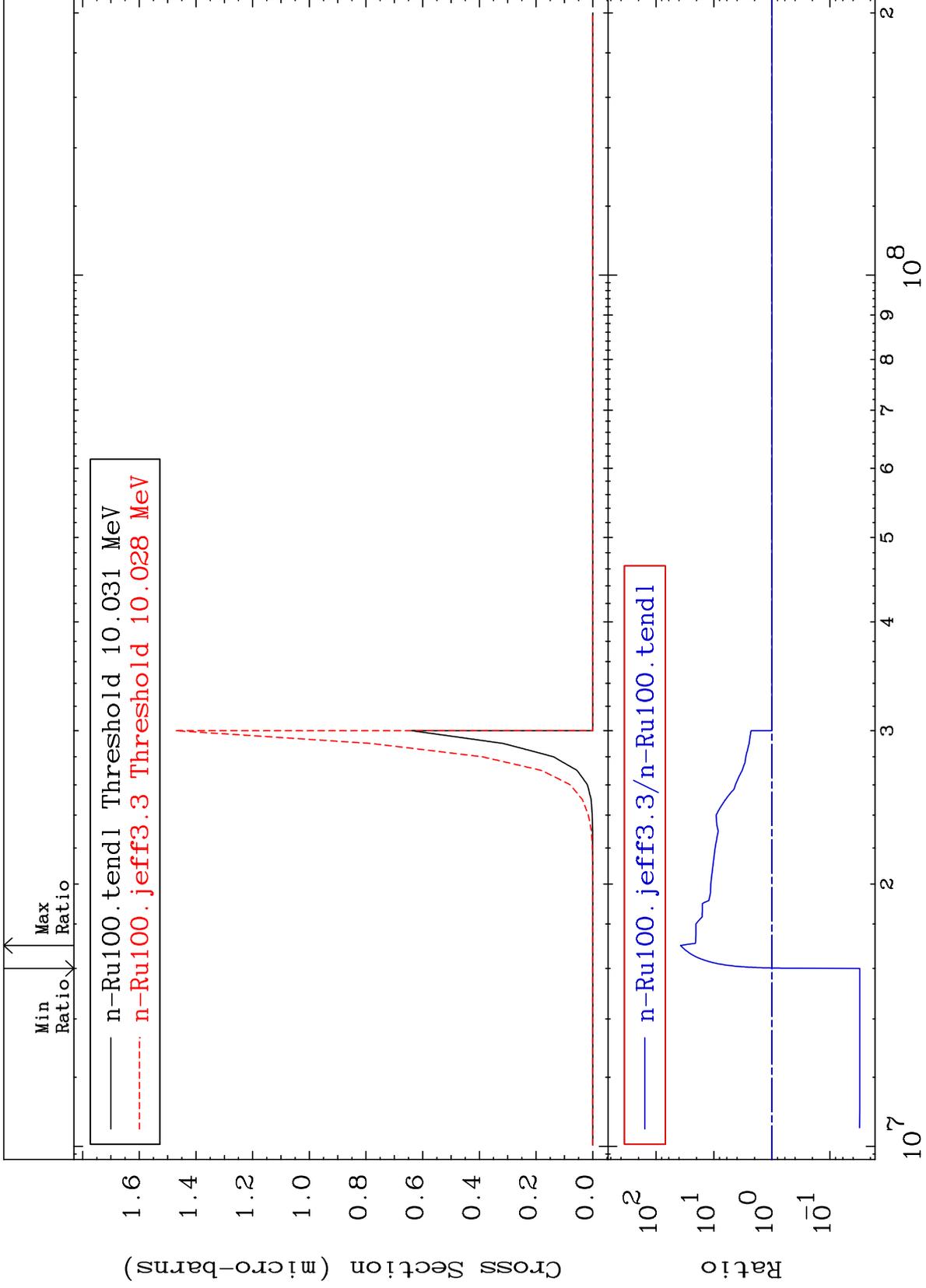
MAT 4437

(n, d)  $\alpha$

44-Ru-100

Cross Section

-96.94 To 3673. %



59

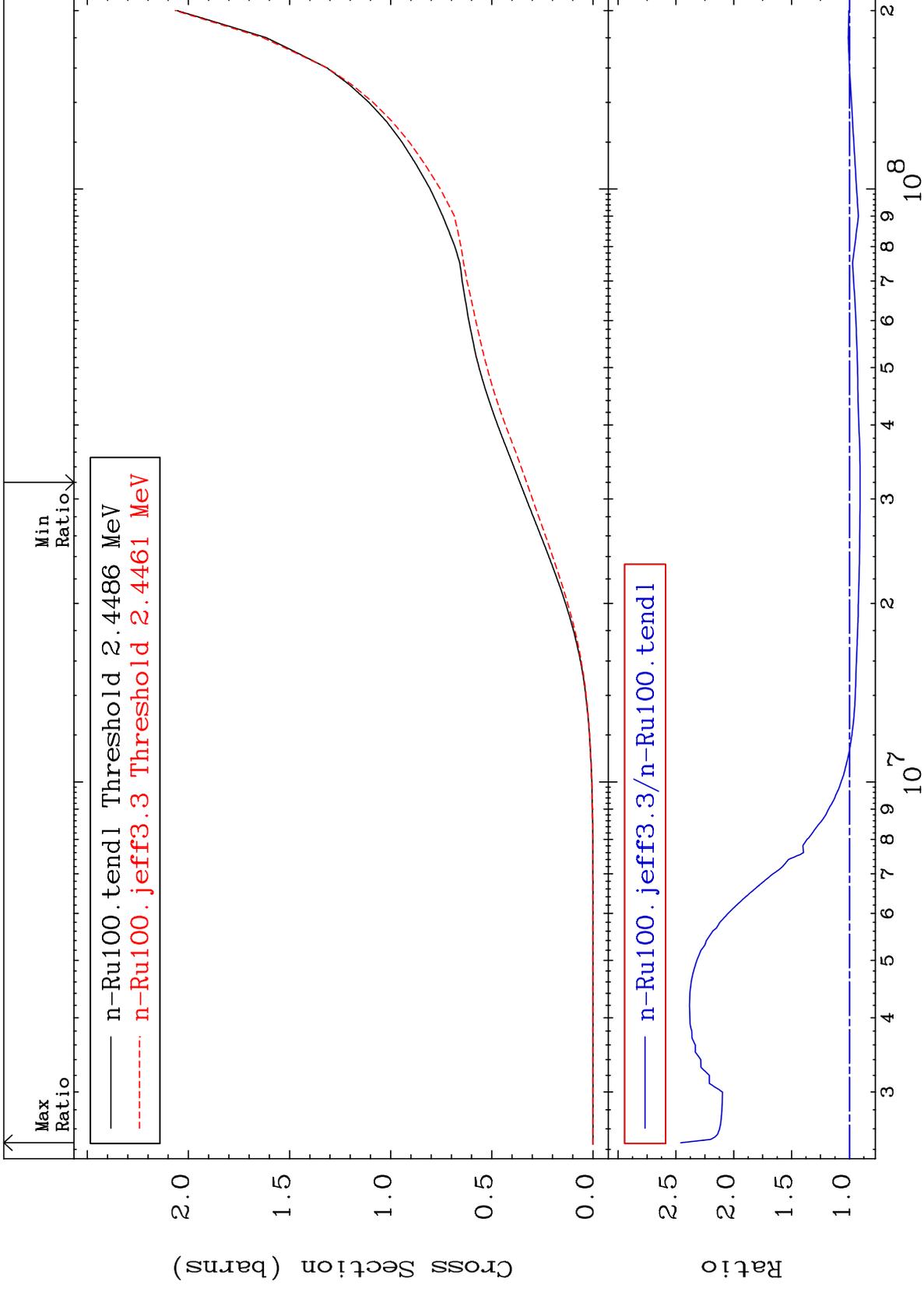
Incident Energy (eV)

44-Ru-100

MAT 4437

Hydrogen Production  
Cross Section

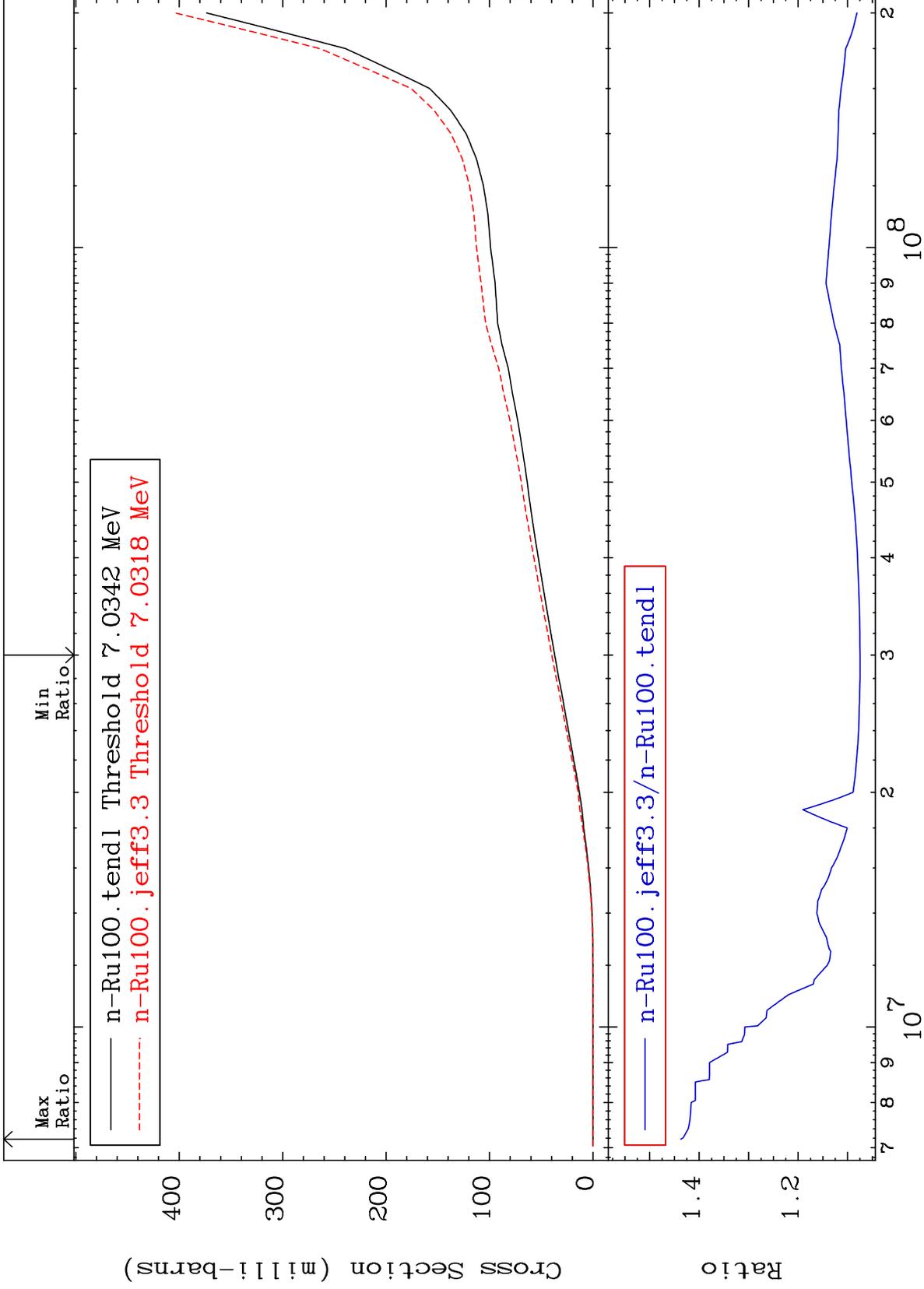
44-Ru-100  
-9.237 To 145.8 %



MAT 4437

Deuterium Production  
Cross Section

44-Ru-100  
7.478 To 43.72 %



61

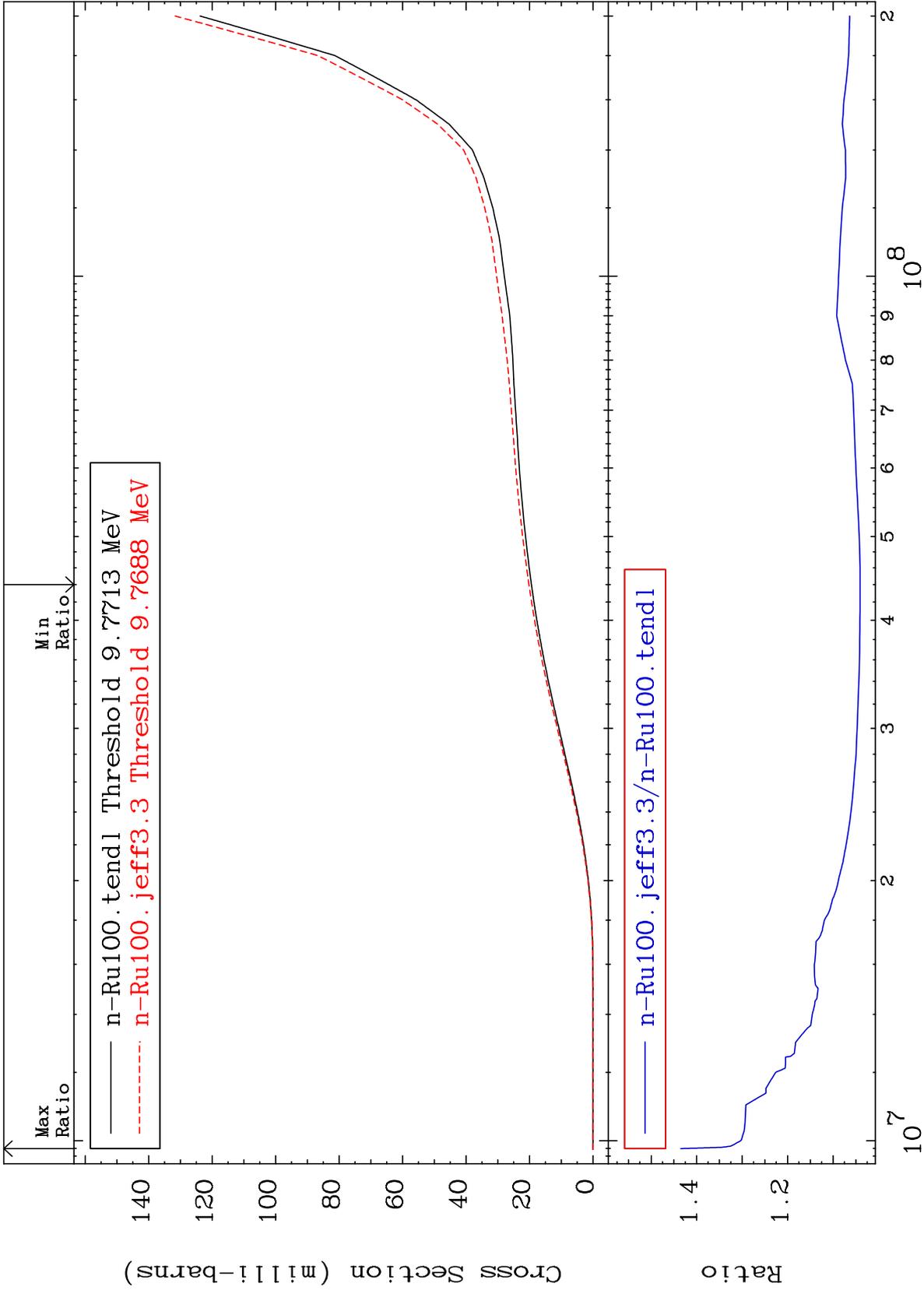
Incident Energy (eV)

44-Ru-100

MAT 4437

Tritium Production  
Cross Section

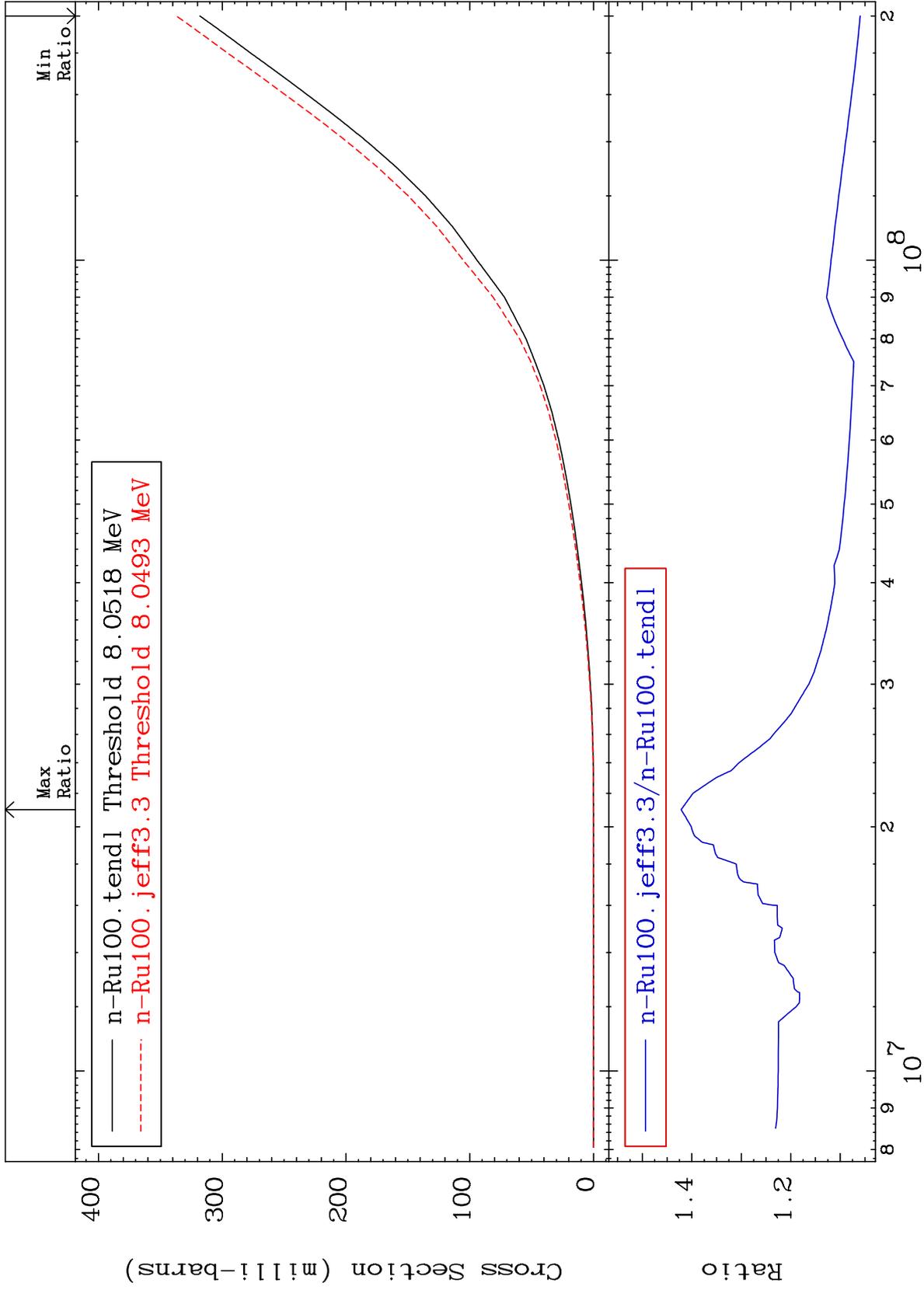
44-Ru-100  
4.062 To 43.49 %



MAT 4437

He-3 Production  
Cross Section

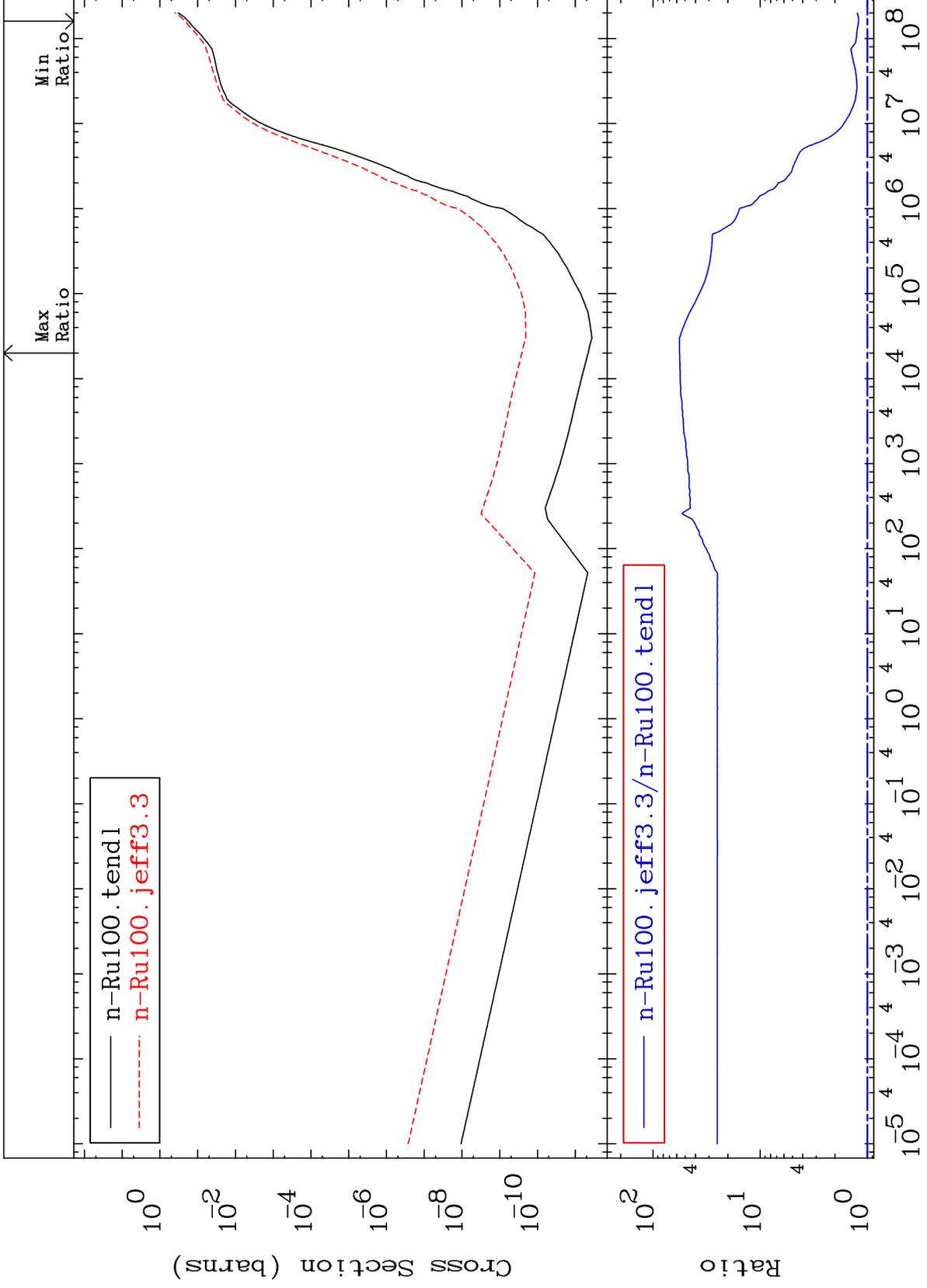
44-Ru-100  
5.957 To 42.15 %



MAT 4437

He-4 Production  
Cross Section

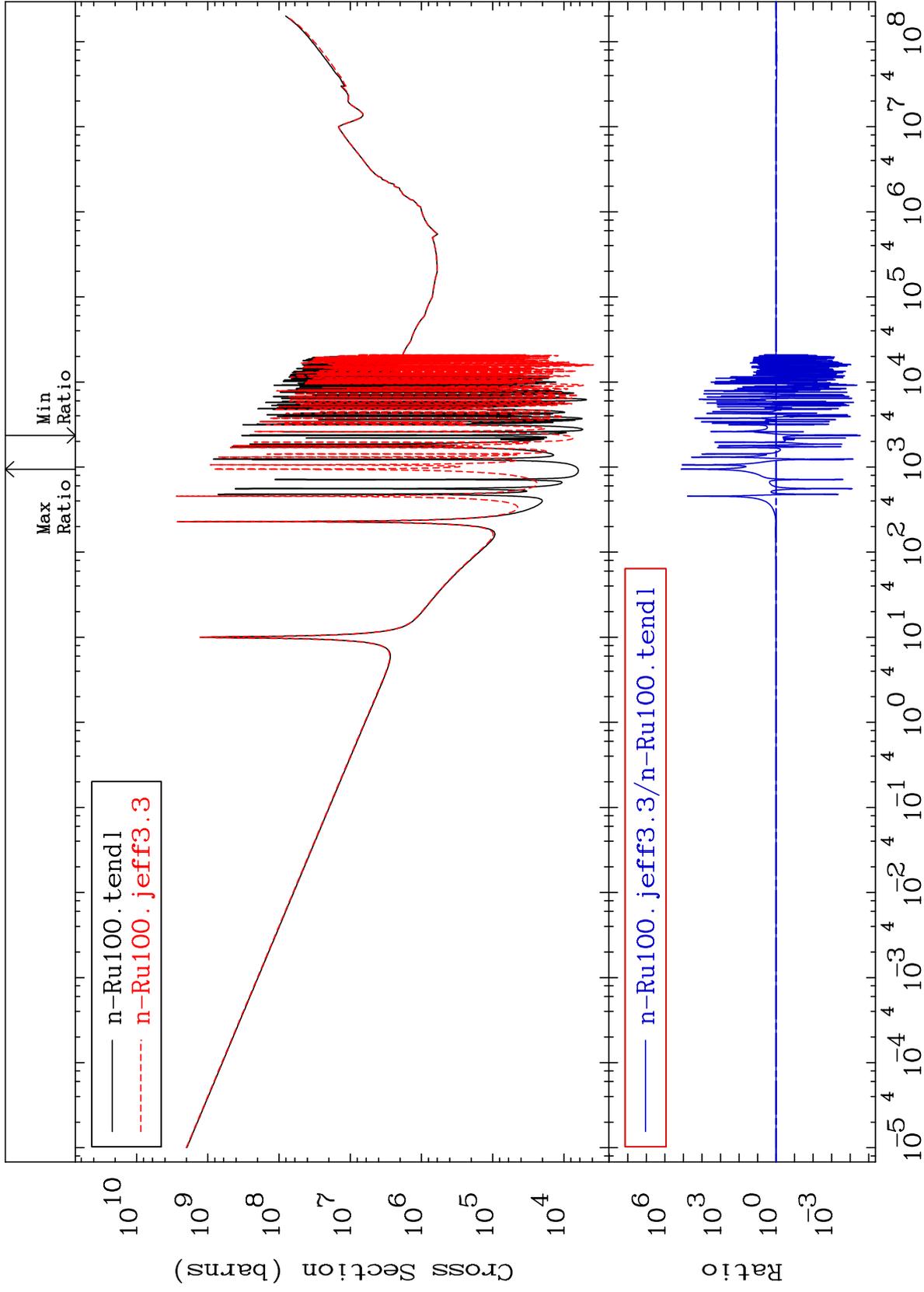
44-Ru-100  
20.69 To 5545. %

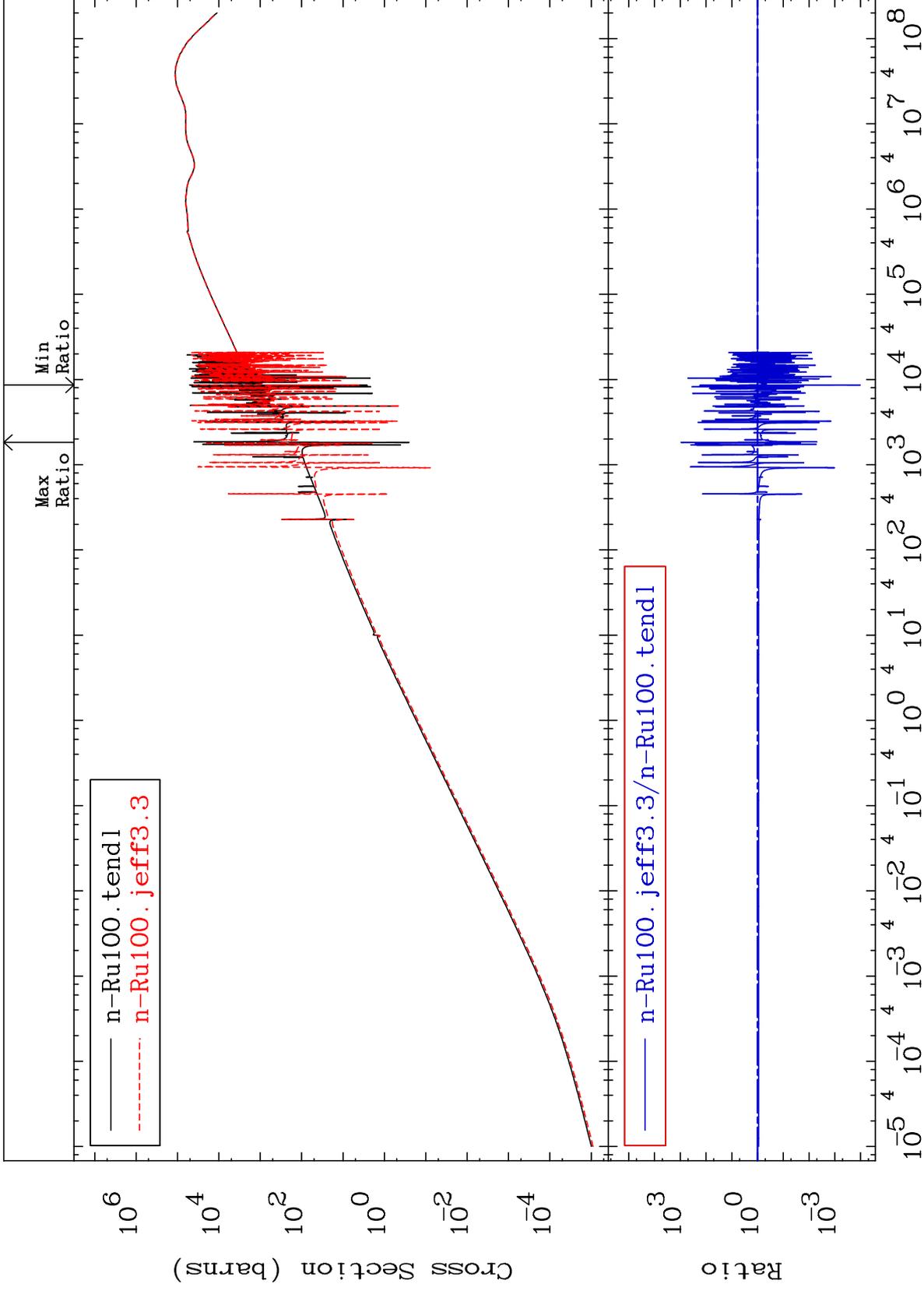


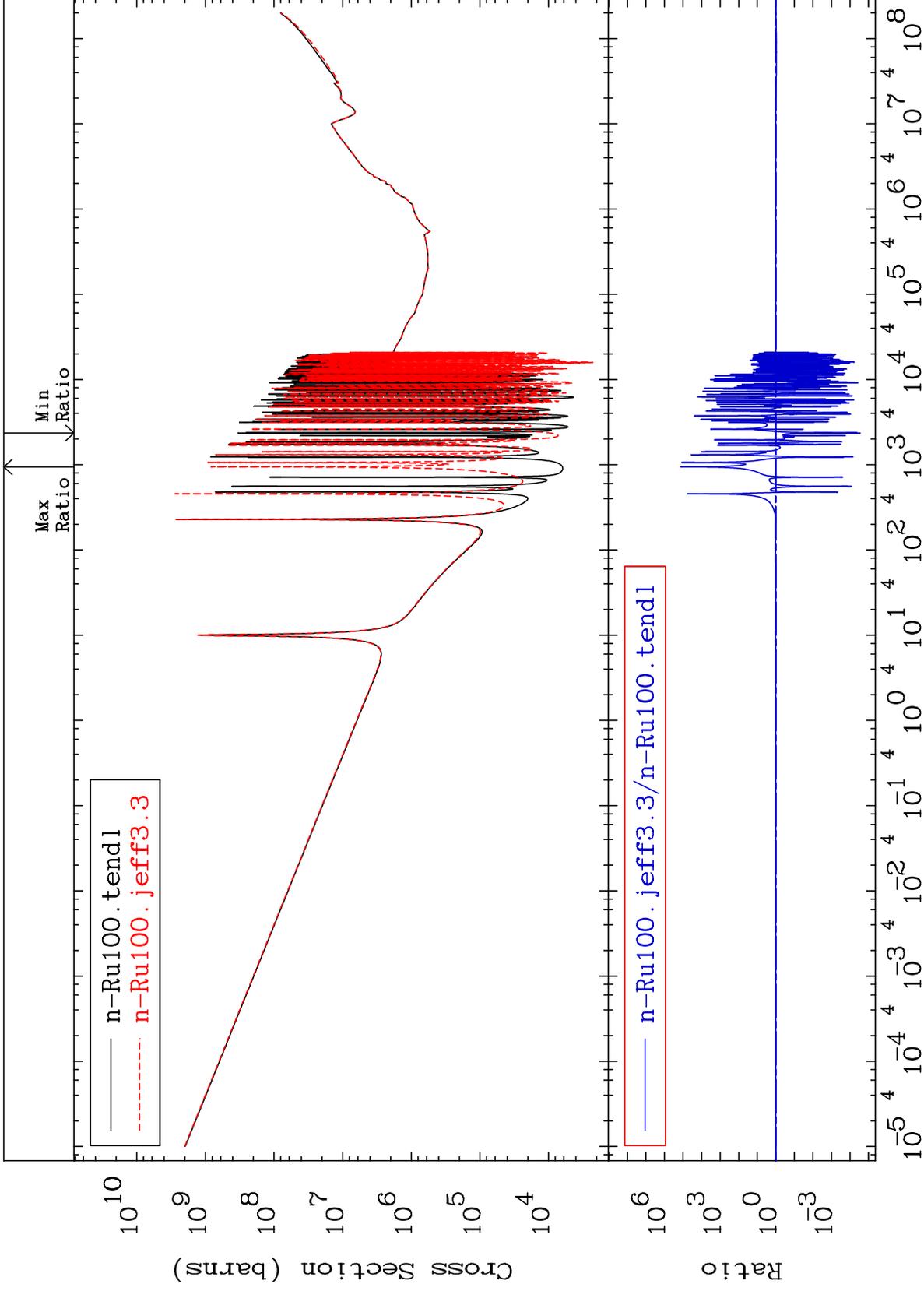
64

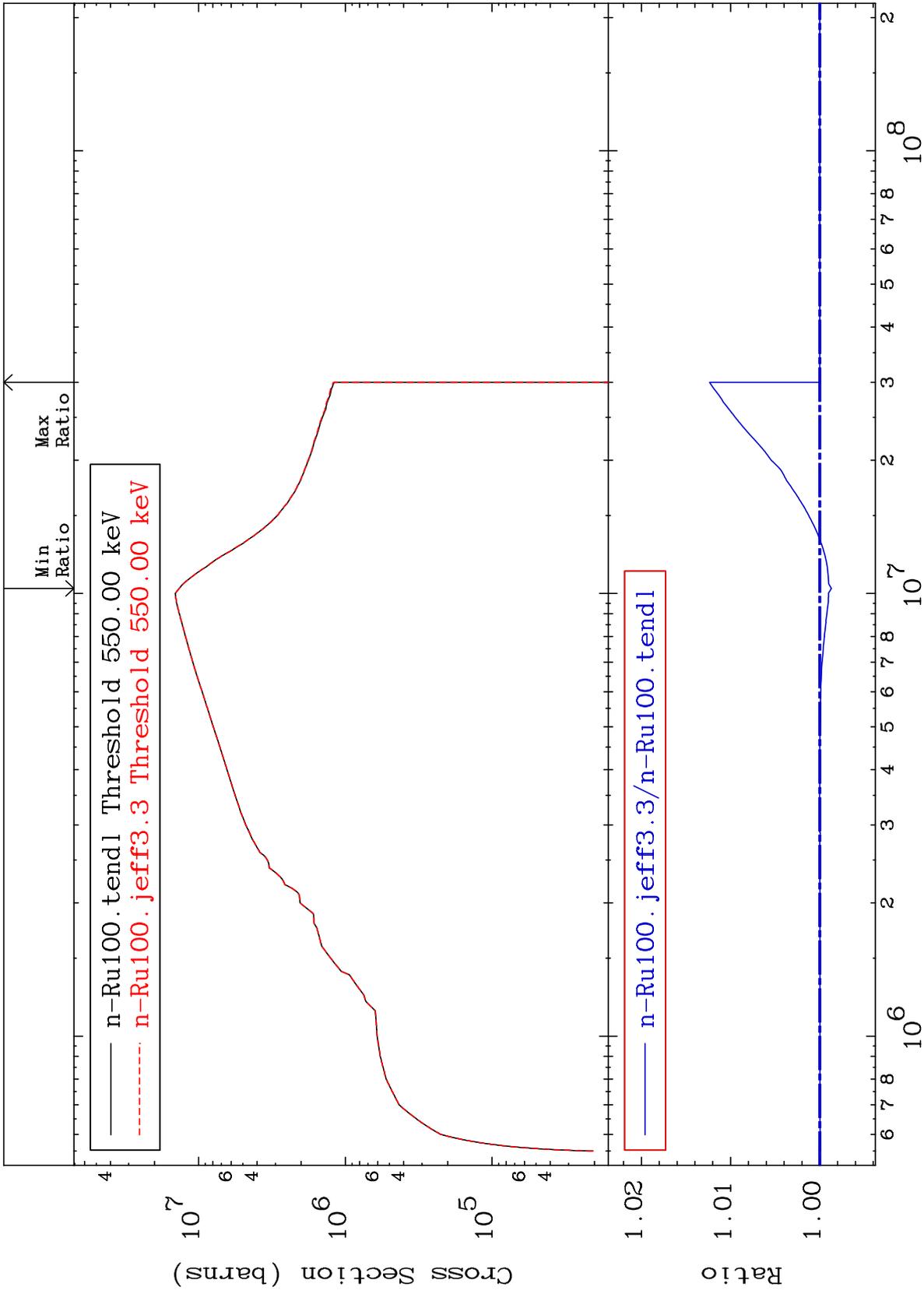
Incident Energy (eV)

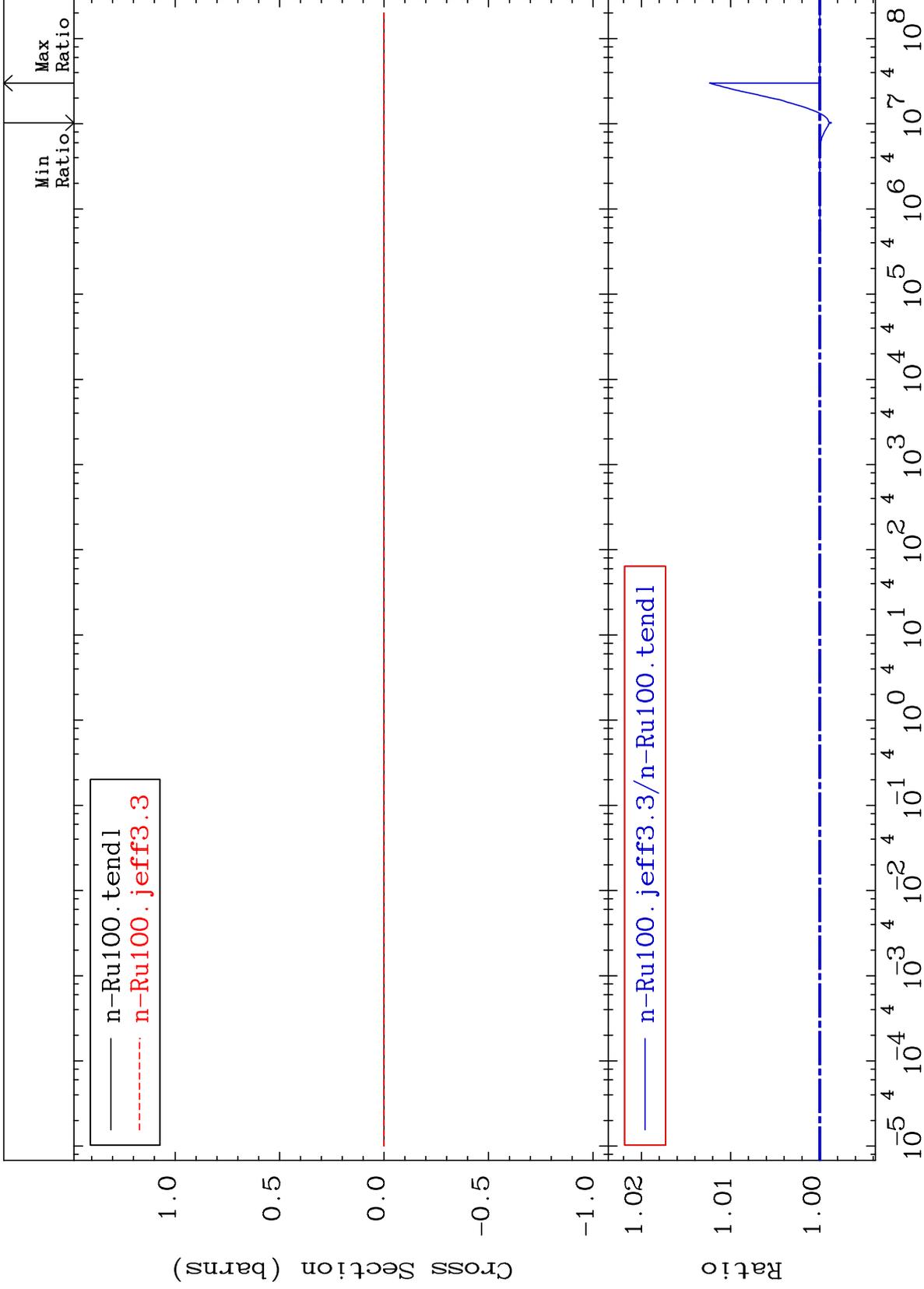
44-Ru-100







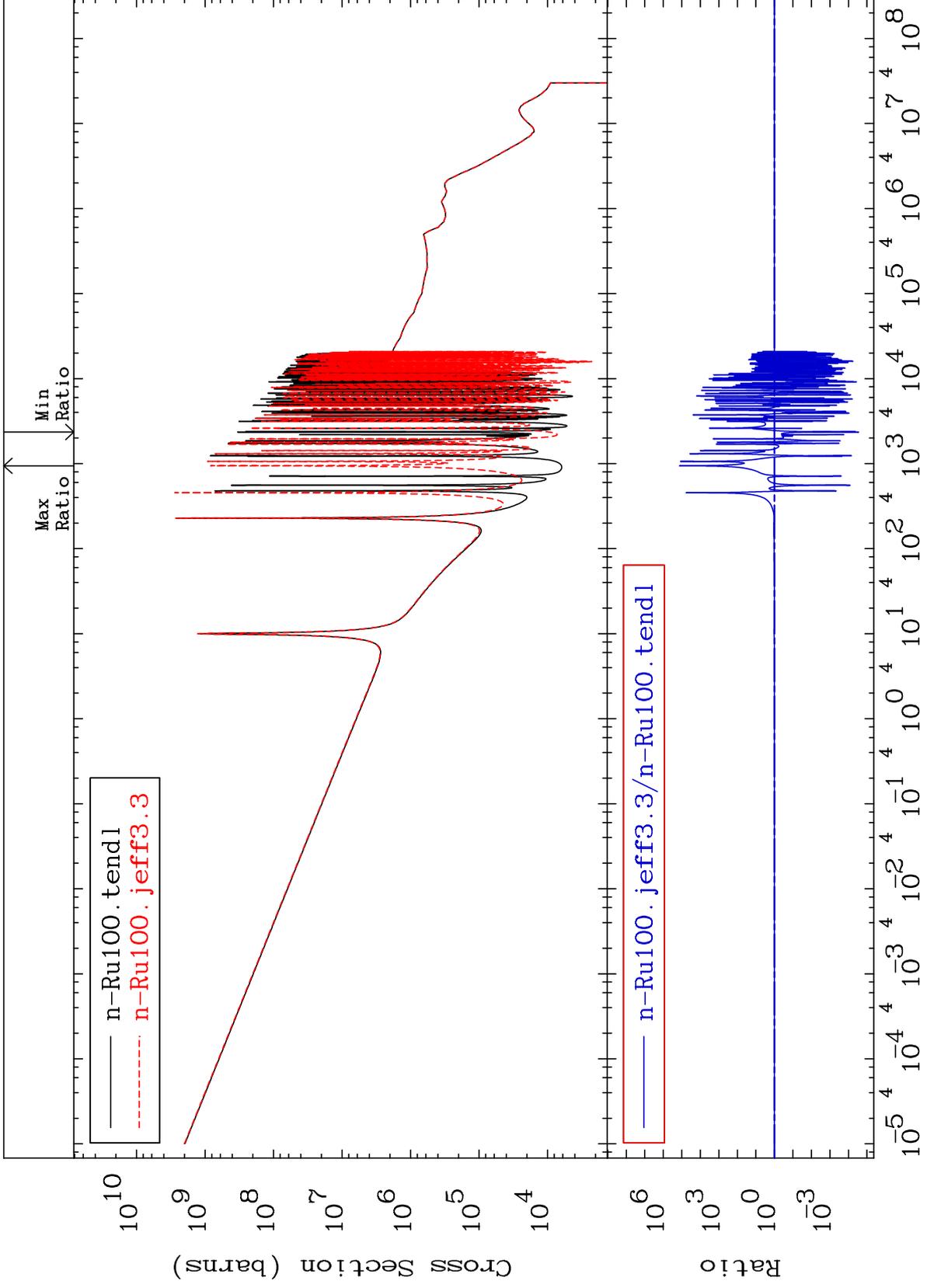




MAT 4437

Kerma capture (mt102)  
Cross Section

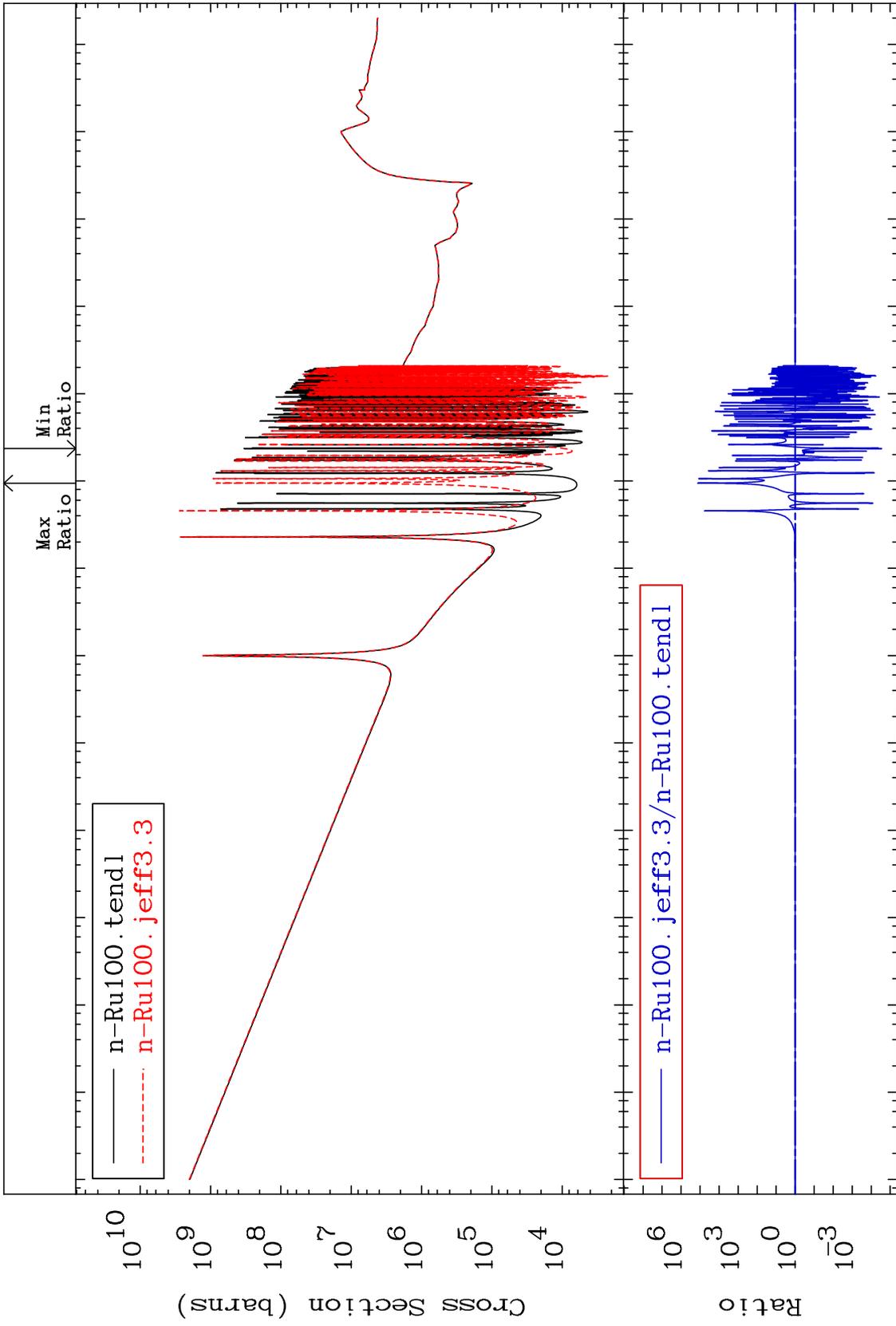
44-Ru-100  
-100.0 To 9999. %

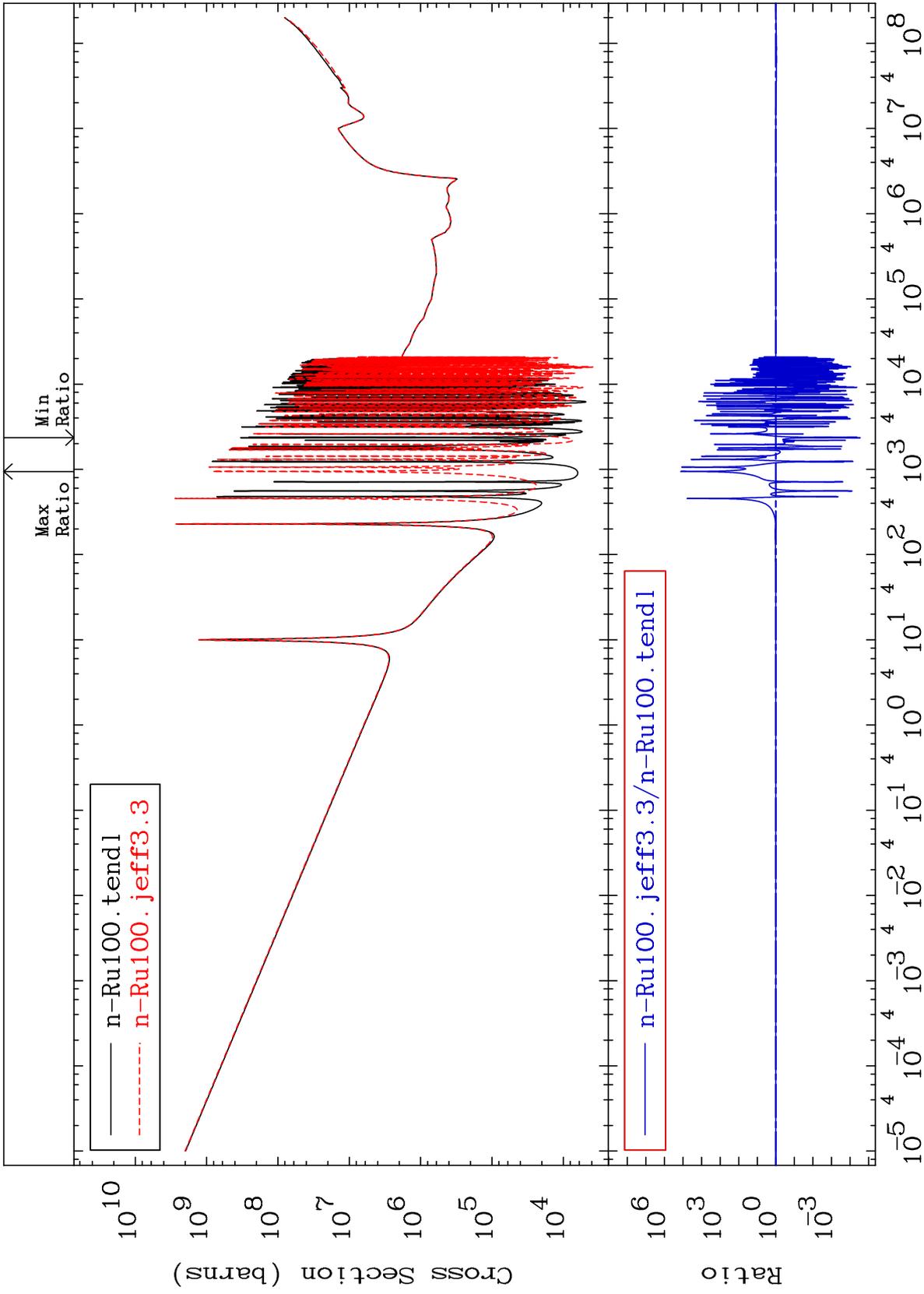


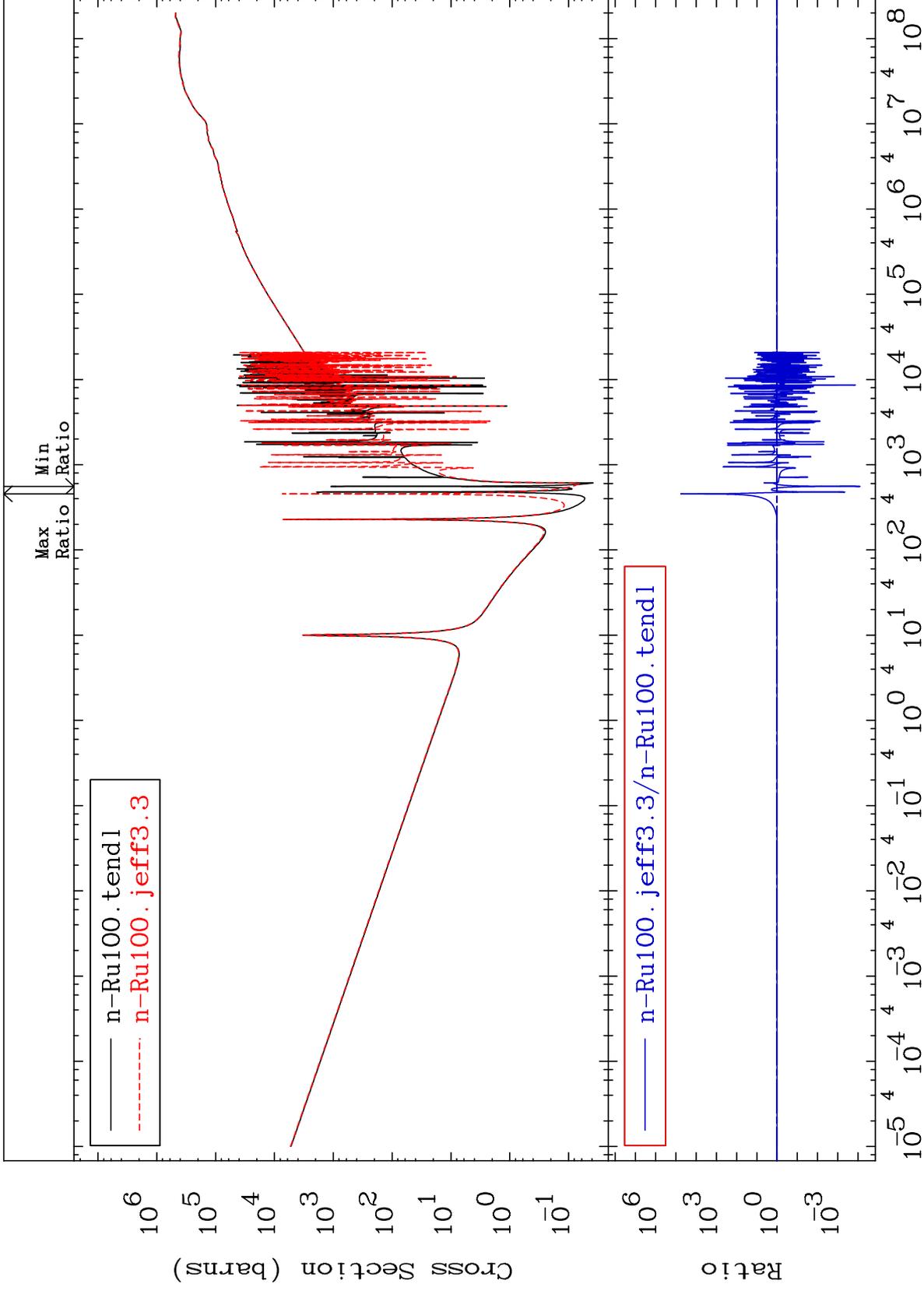
70

Incident Energy (eV)

44-Ru-100



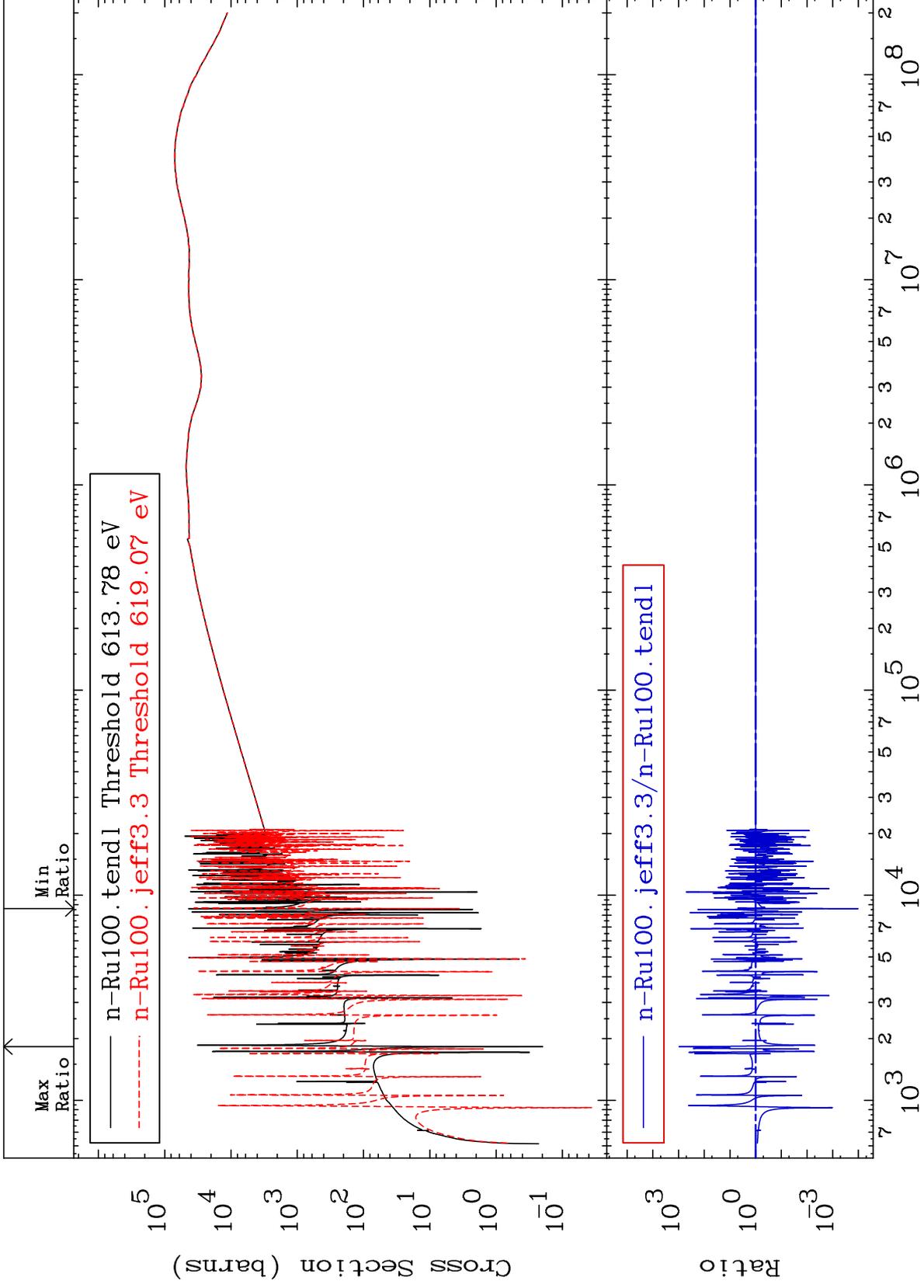




MAT 4437

Dpa elastic (mt2)  
Cross Section

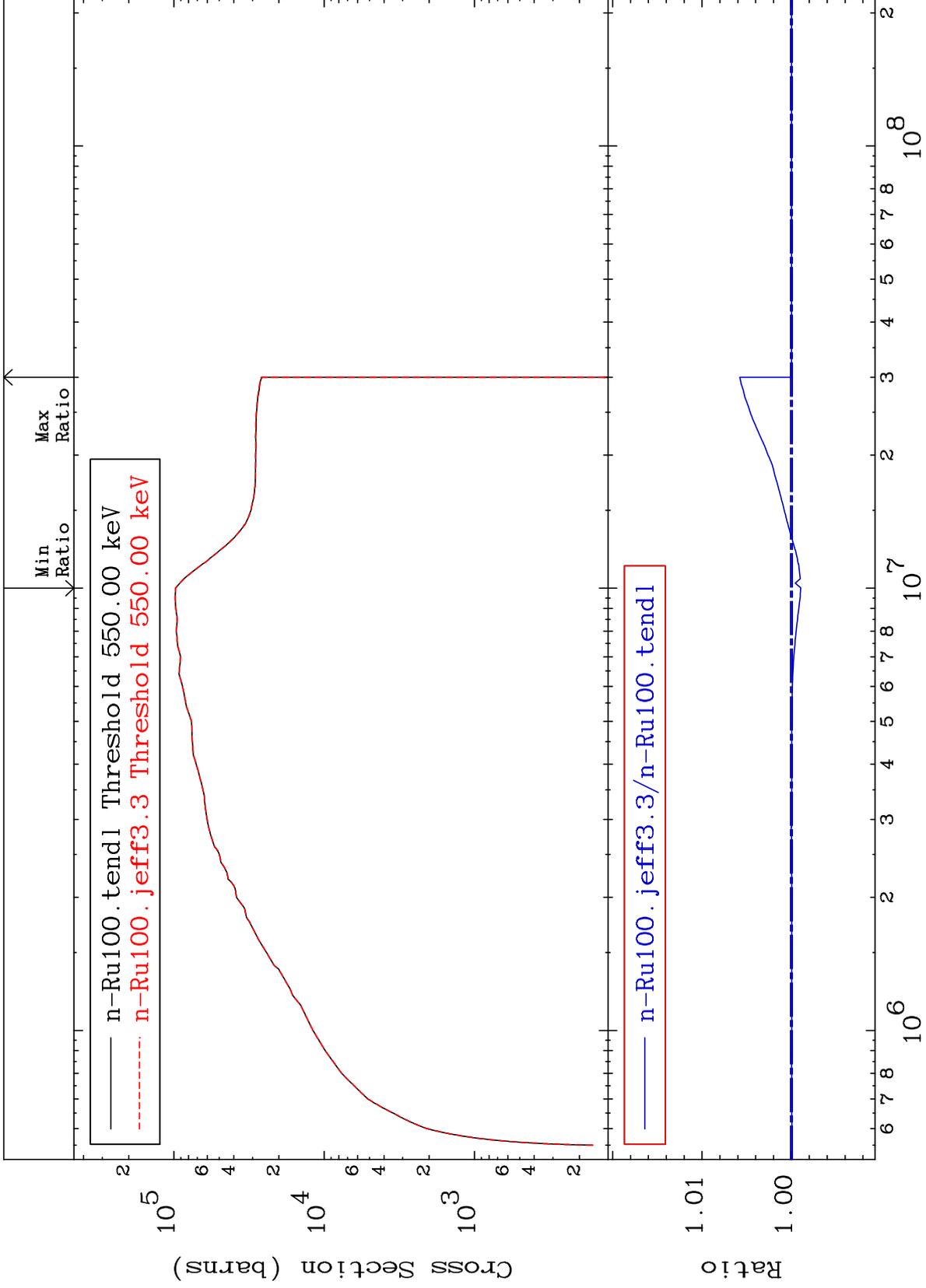
44-Ru-100  
-99.99 To 9999. %

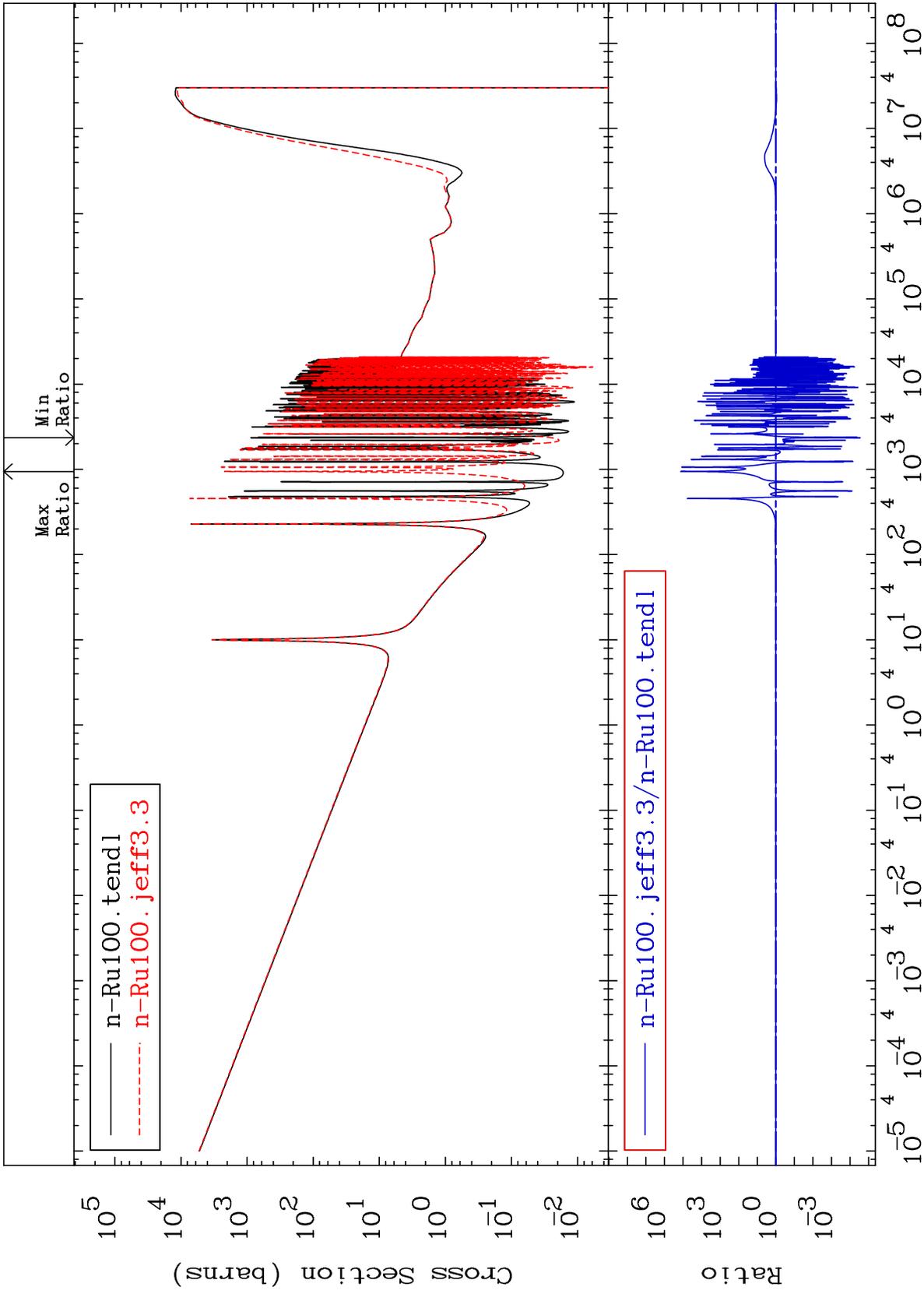


74

Incident Energy (eV)

44-Ru-100



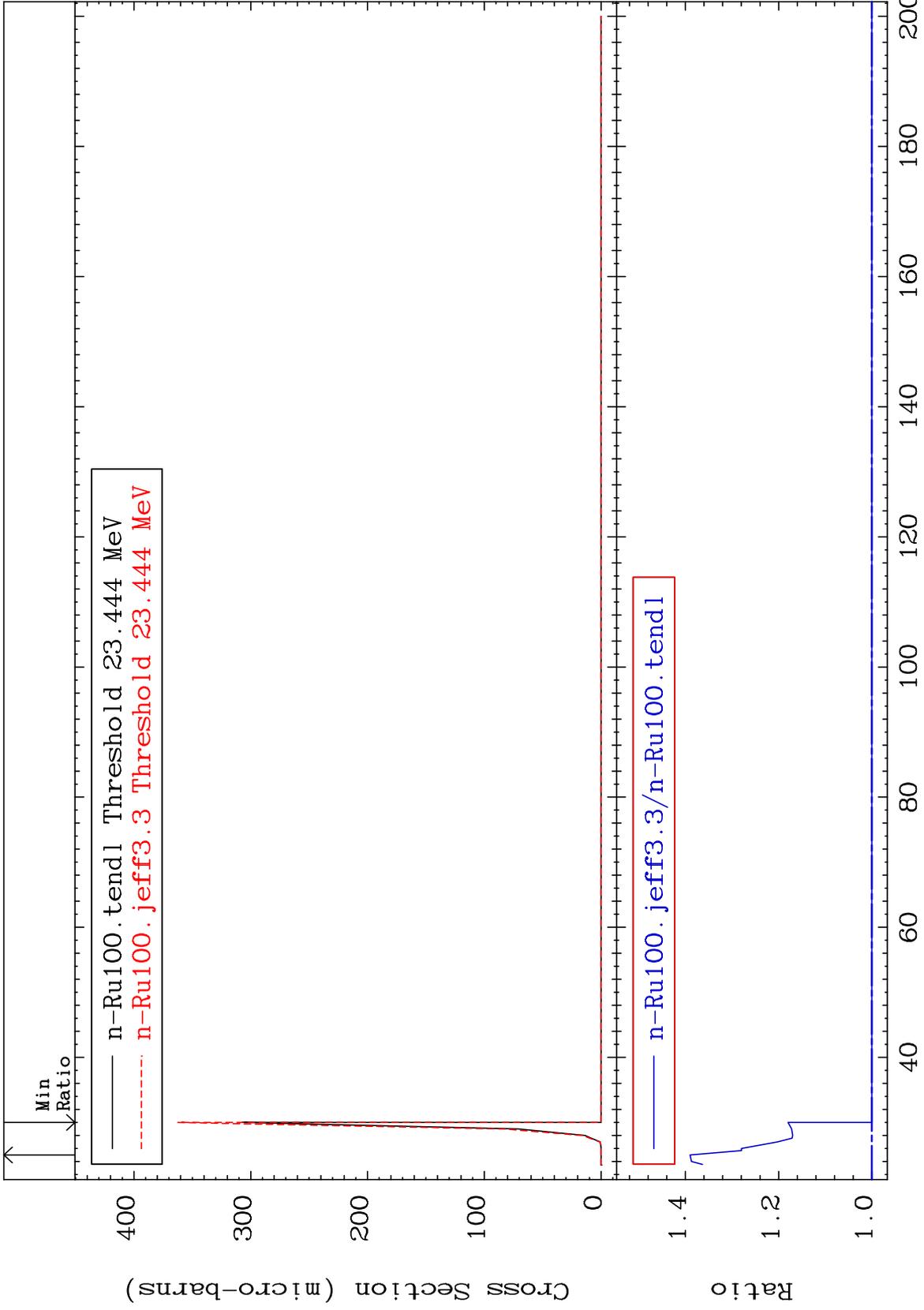


MAT 4437

(n,2n) d:43-Tc-97g

44-Ru-100

Radionuclide Production Cross Section 0.000 To 39.00 %

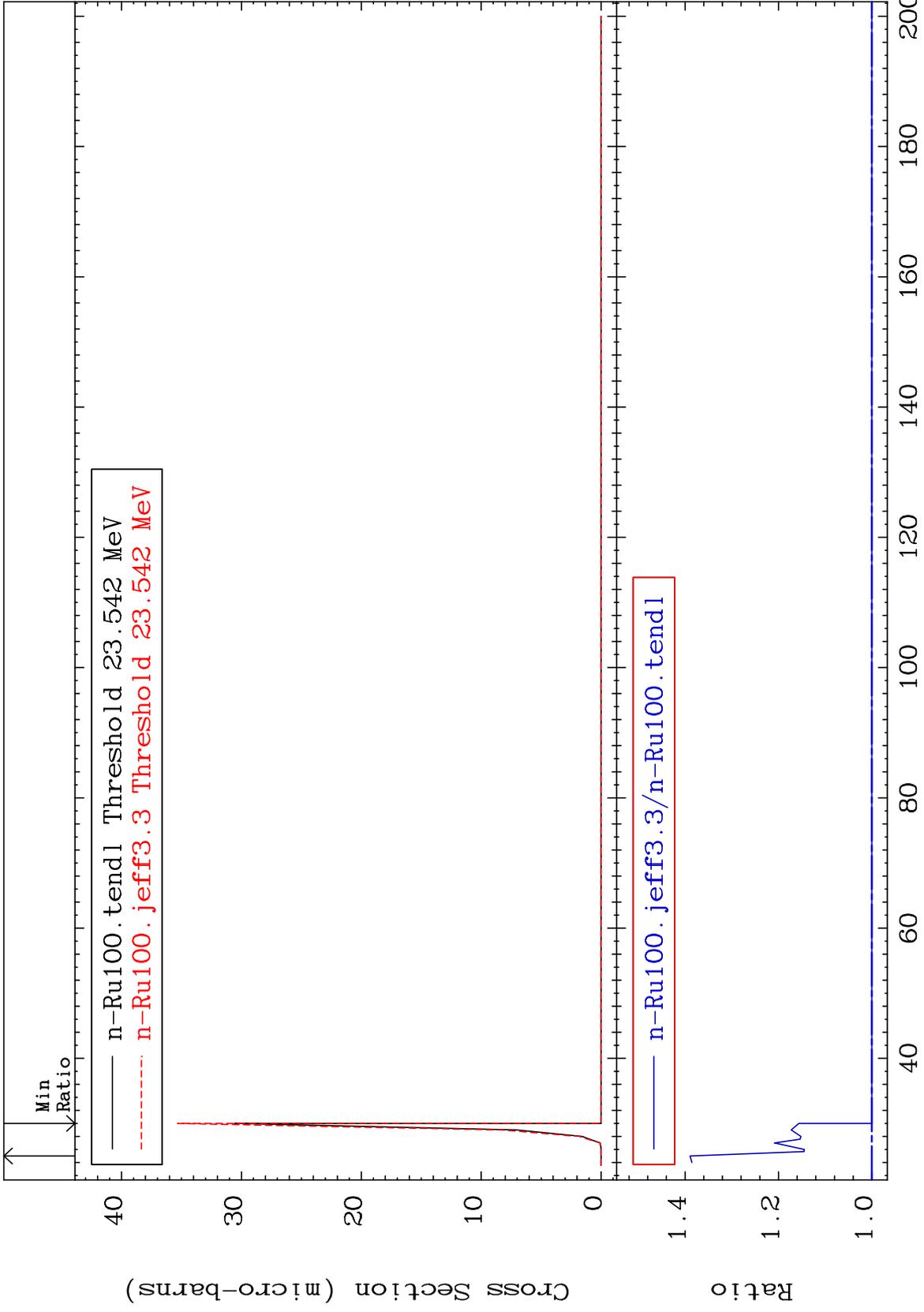


MAT 4437

(n,2n) d:43-Tc-97m1

44-Ru-100

Radionuclide Production Cross Section 0.000 To 38.93 %



78

Incident Energy (MeV)

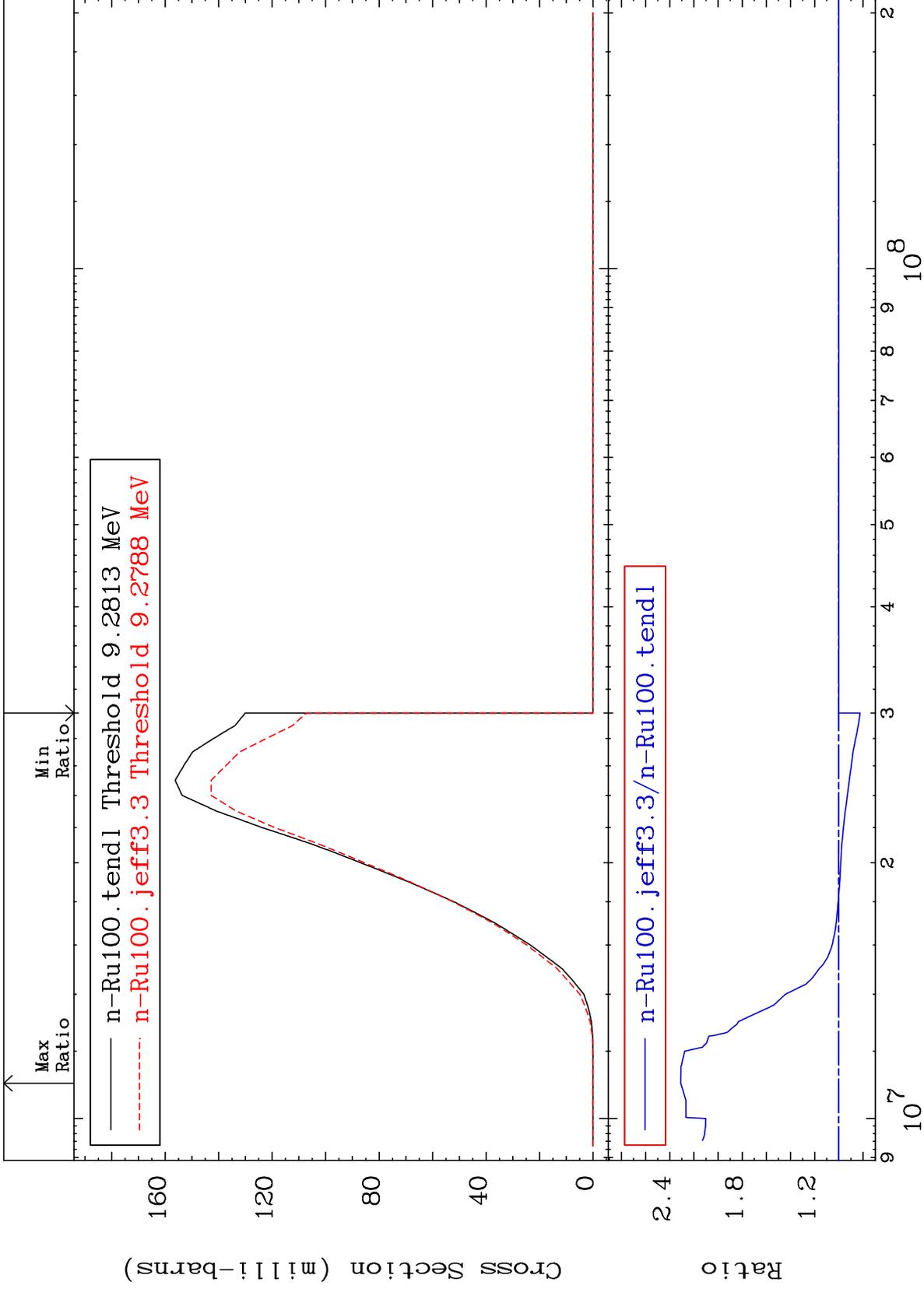
44-Ru-100

MAT 4437

(n, n') p:43-Tc-99g

44-Ru-100

Radionuclide Production Cross Section -17.58 To 130.9 %



79

44-Ru-100

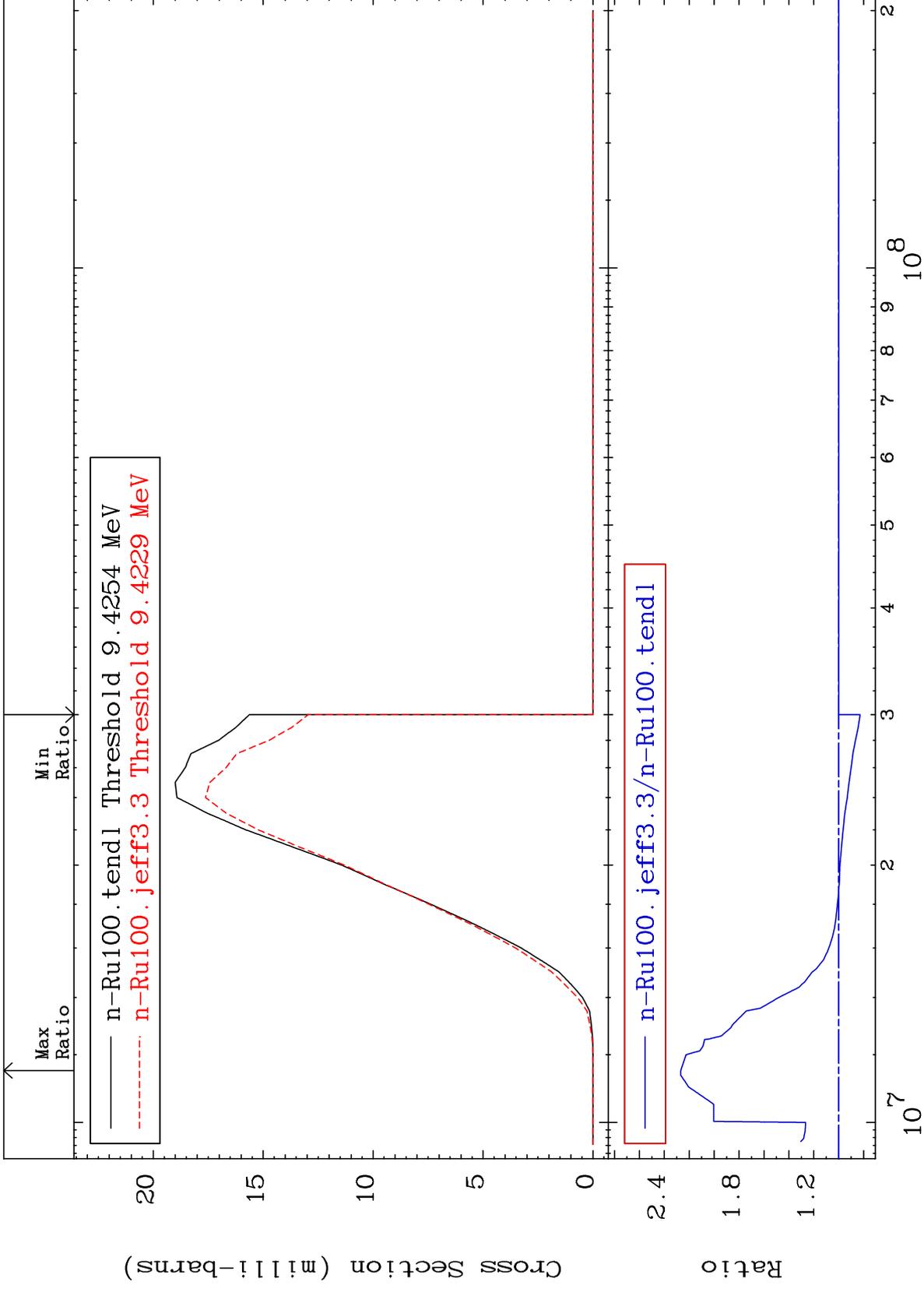
44-Ru-100

MAT 4437

(n, n') p: 43-Tc-99m2

44-Ru-100

Radionuclide Production Cross Section -17.17 To 126.8 %

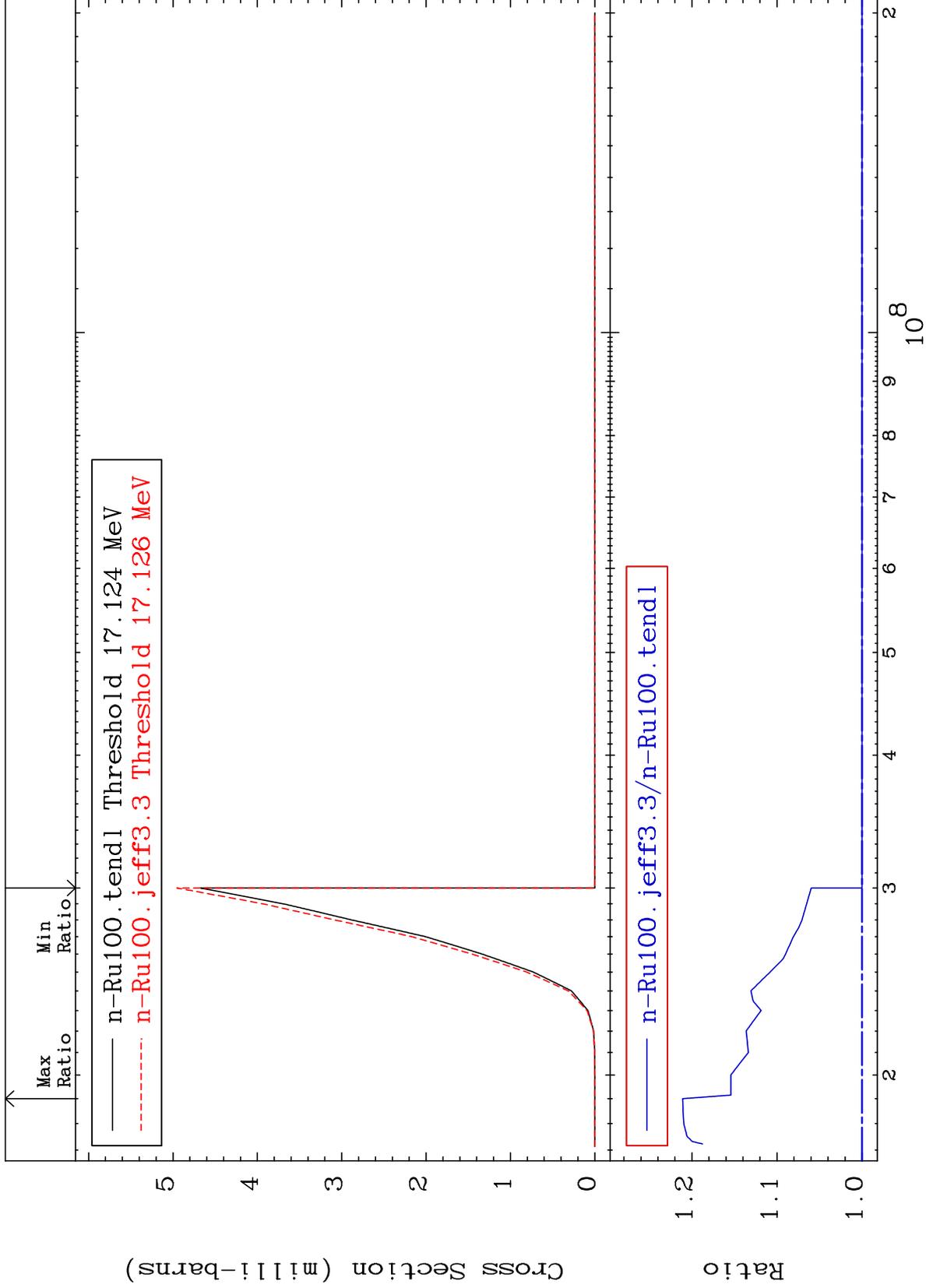


MAT 4437

(n, n') t:43-Tc-97g

44-Ru-100

Radionuclide Production Cross Section 0.000 To 21.09 %

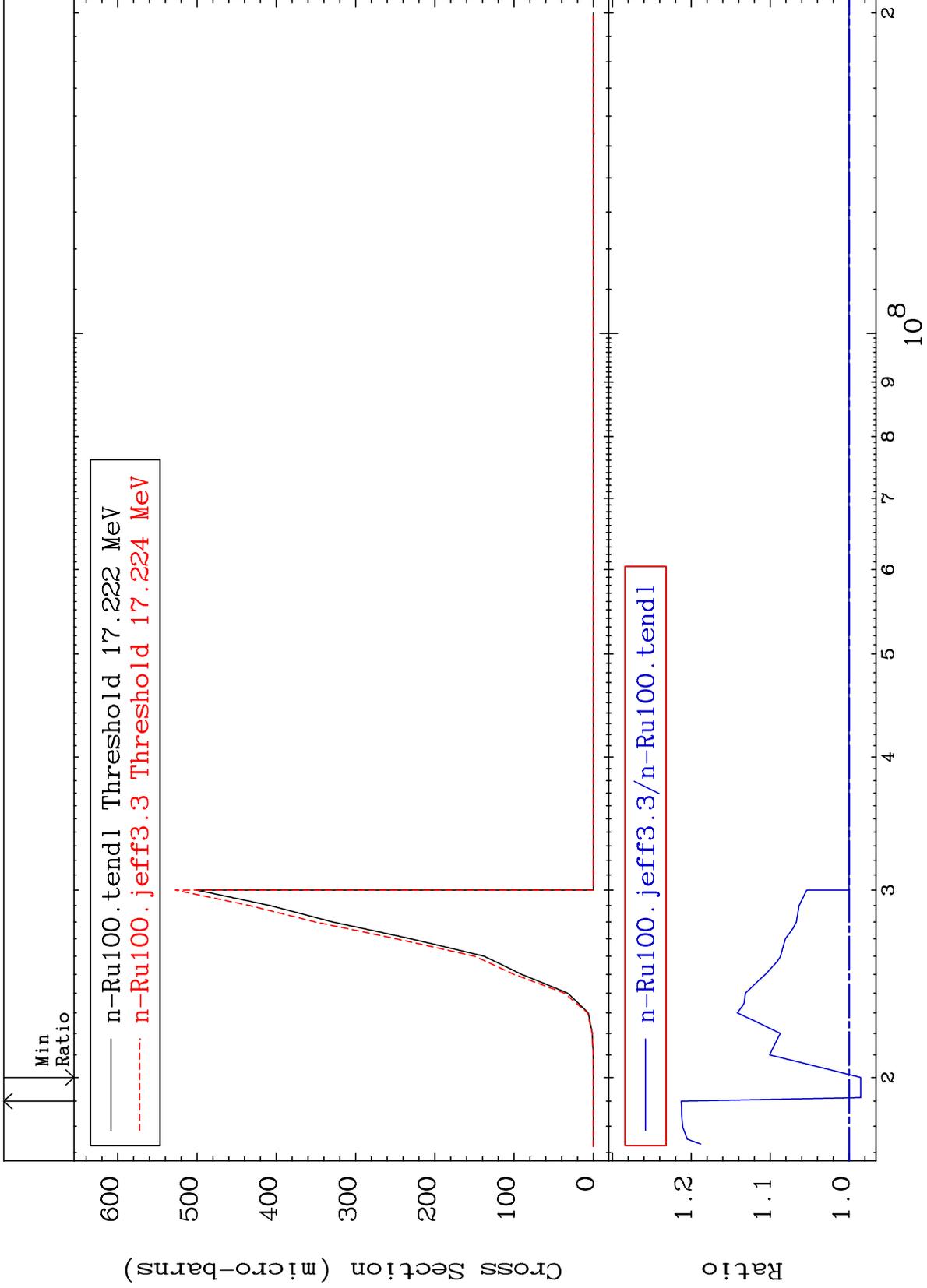


MAT 4437

(n, n') t:43-Tc-97m1

44-Ru-100

Radionuclide Production Cross Section -1.473 To 21.27 %



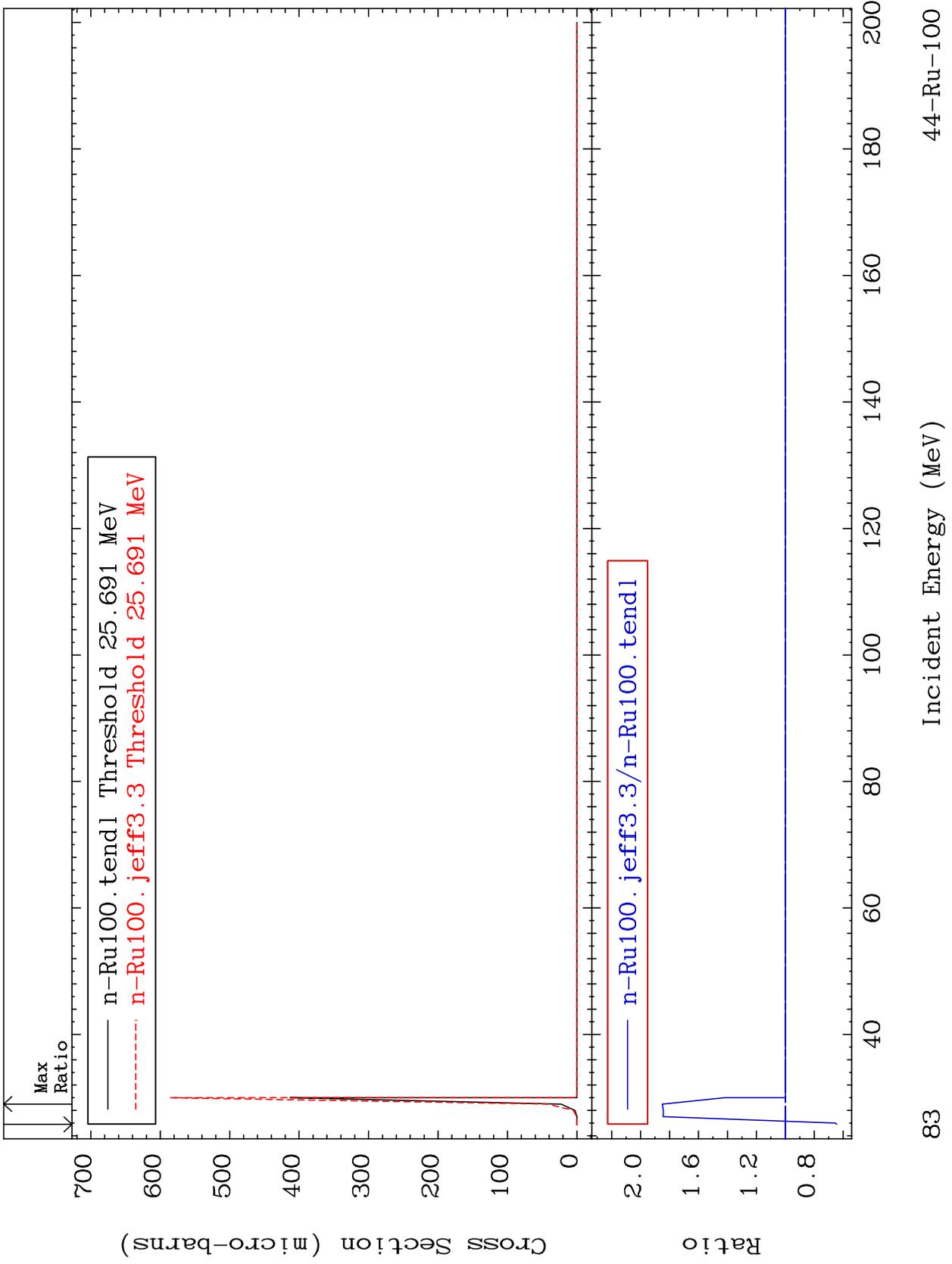
MAT 4437

(n,3n) p:43-Tc-97g

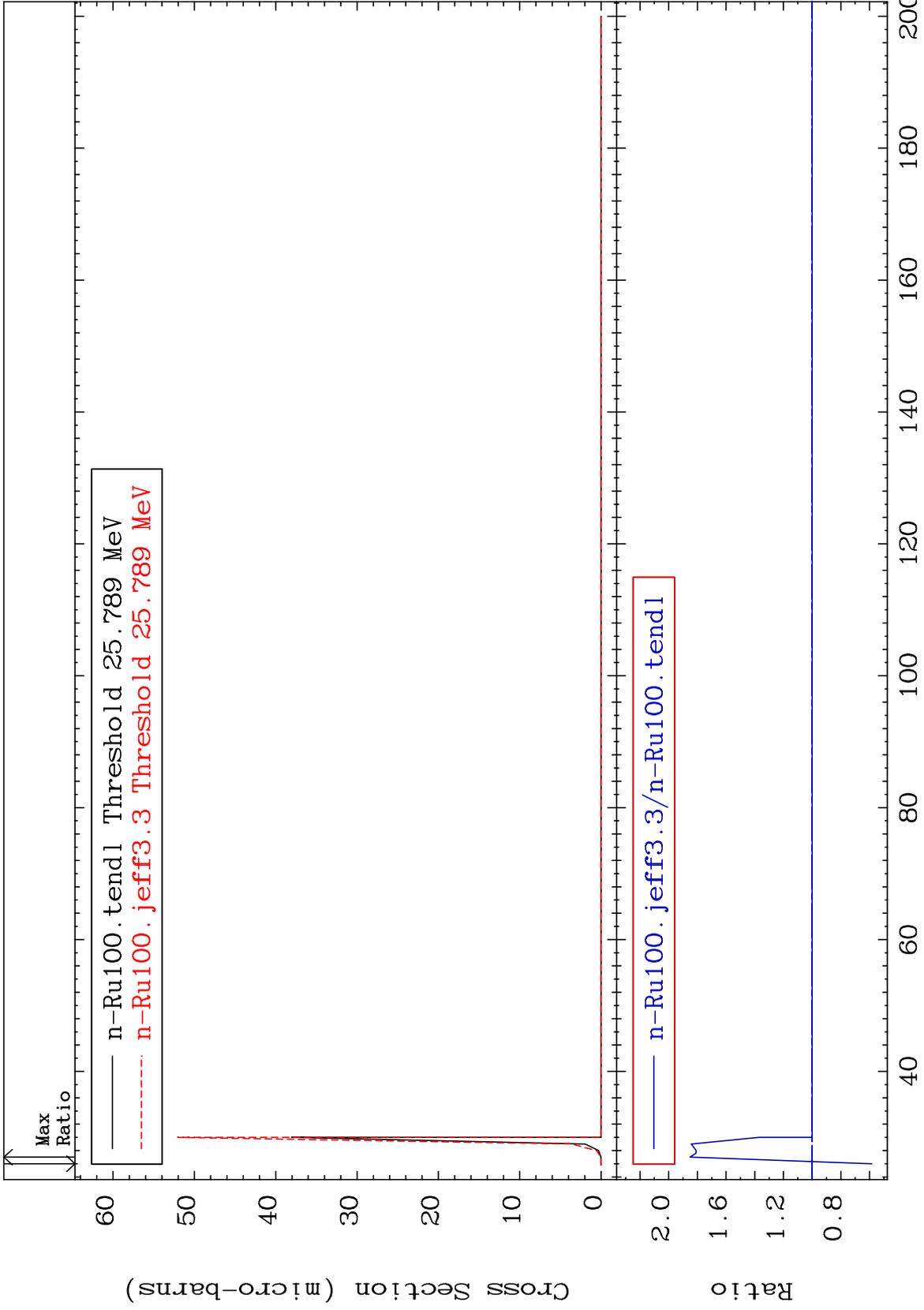
44-Ru-100

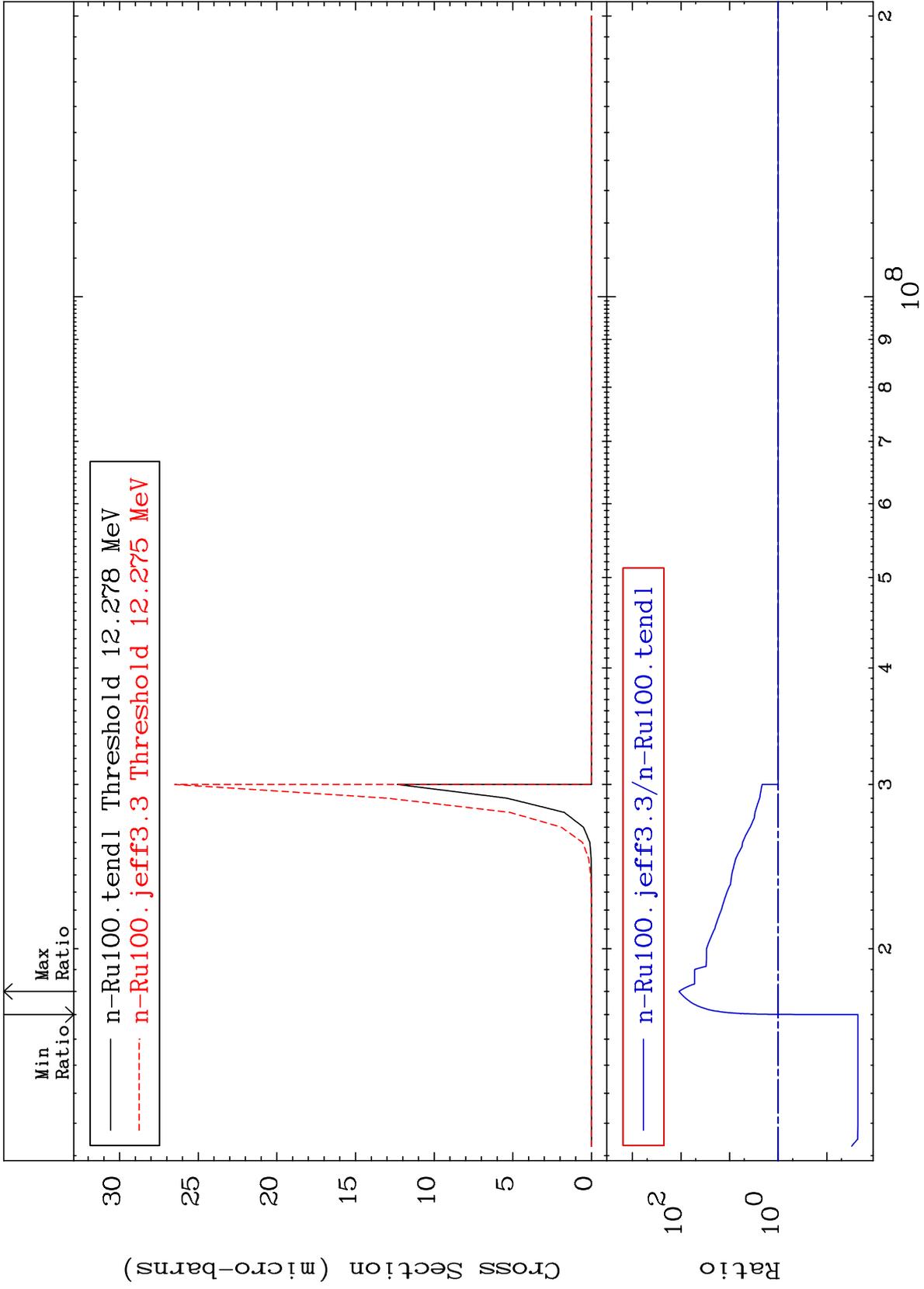
Radionuclide Production Cross Section

-35.59 To 84.97 %



Radionuclide Production Cross Section -41.61 To 85.09 %



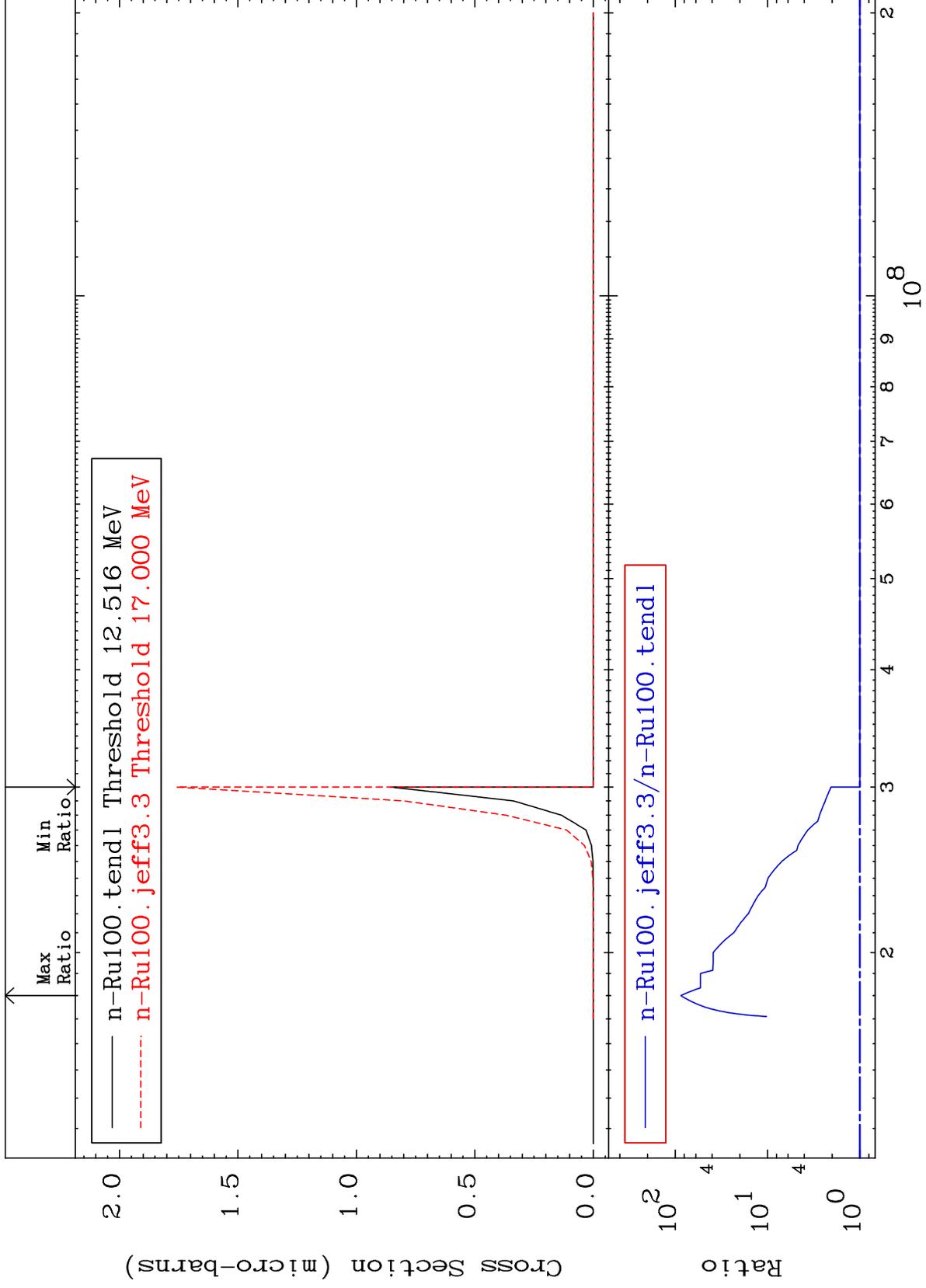


MAT 4437

(n, n') p  $\alpha$ : 41-Nb-95m1

44-Ru-100

Radionuclide Production Cross Section 0.000 To 8594. %

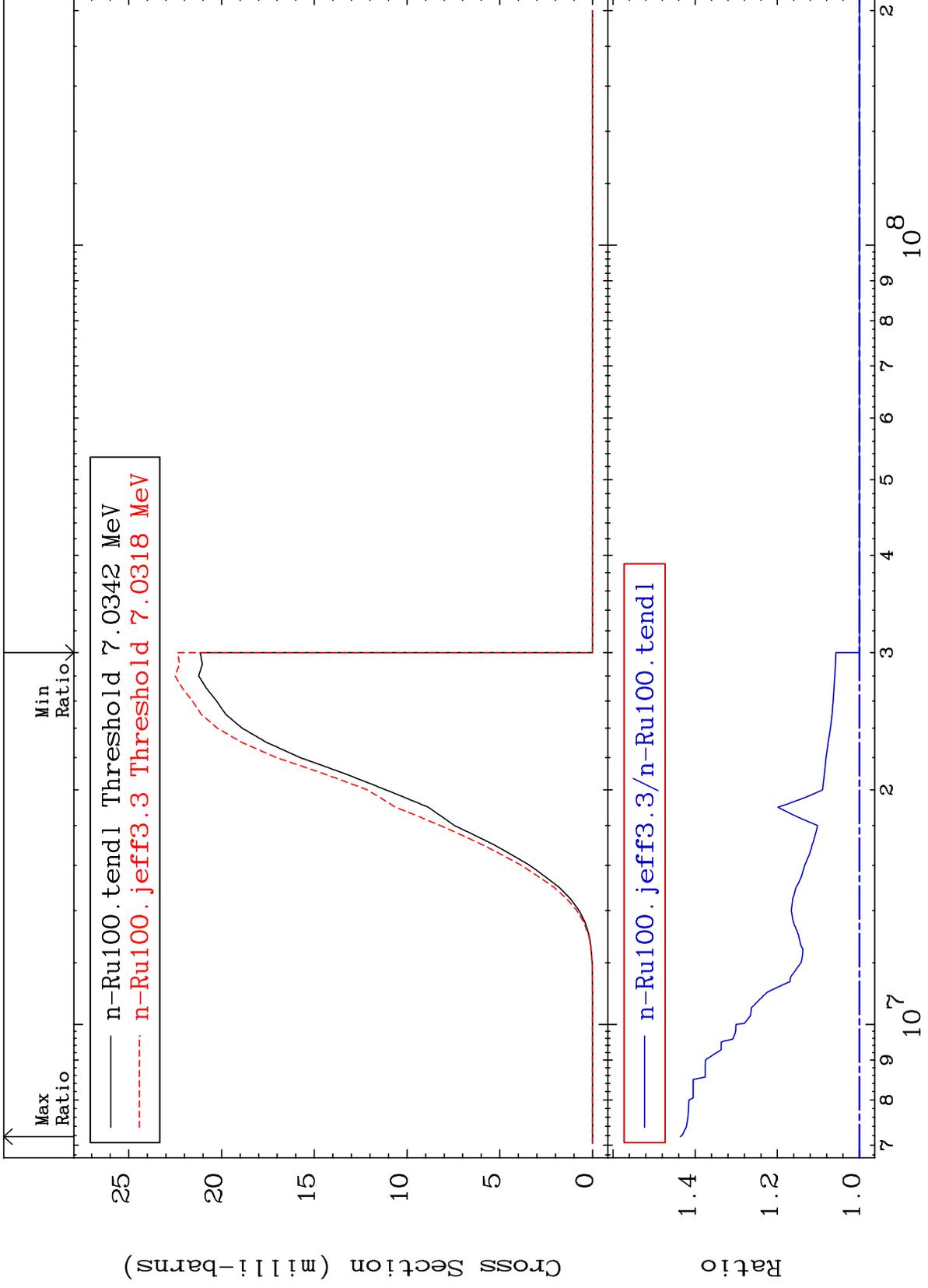


MAT 4437

(n, d) : 43-Tc-99g

44-Ru-100

Radionuclide Production Cross Section 0.000 To 43.62 %



87

Incident Energy (eV)

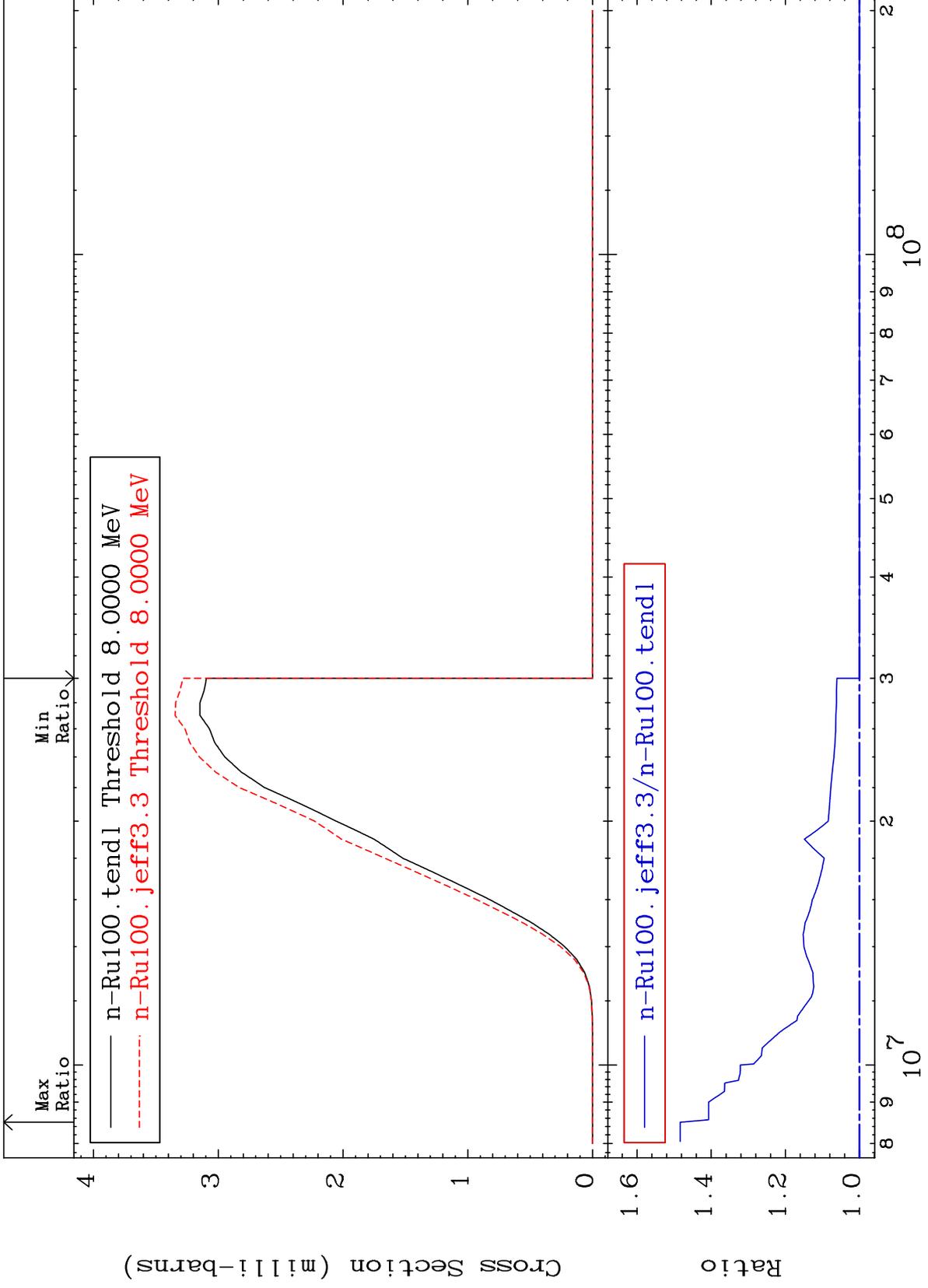
44-Ru-100

MAT 4437

(n, d) : 43-Tc-99m2

44-Ru-100

Radionuclide Production Cross Section 0.000 To 48.34 %



88

Incident Energy (eV)

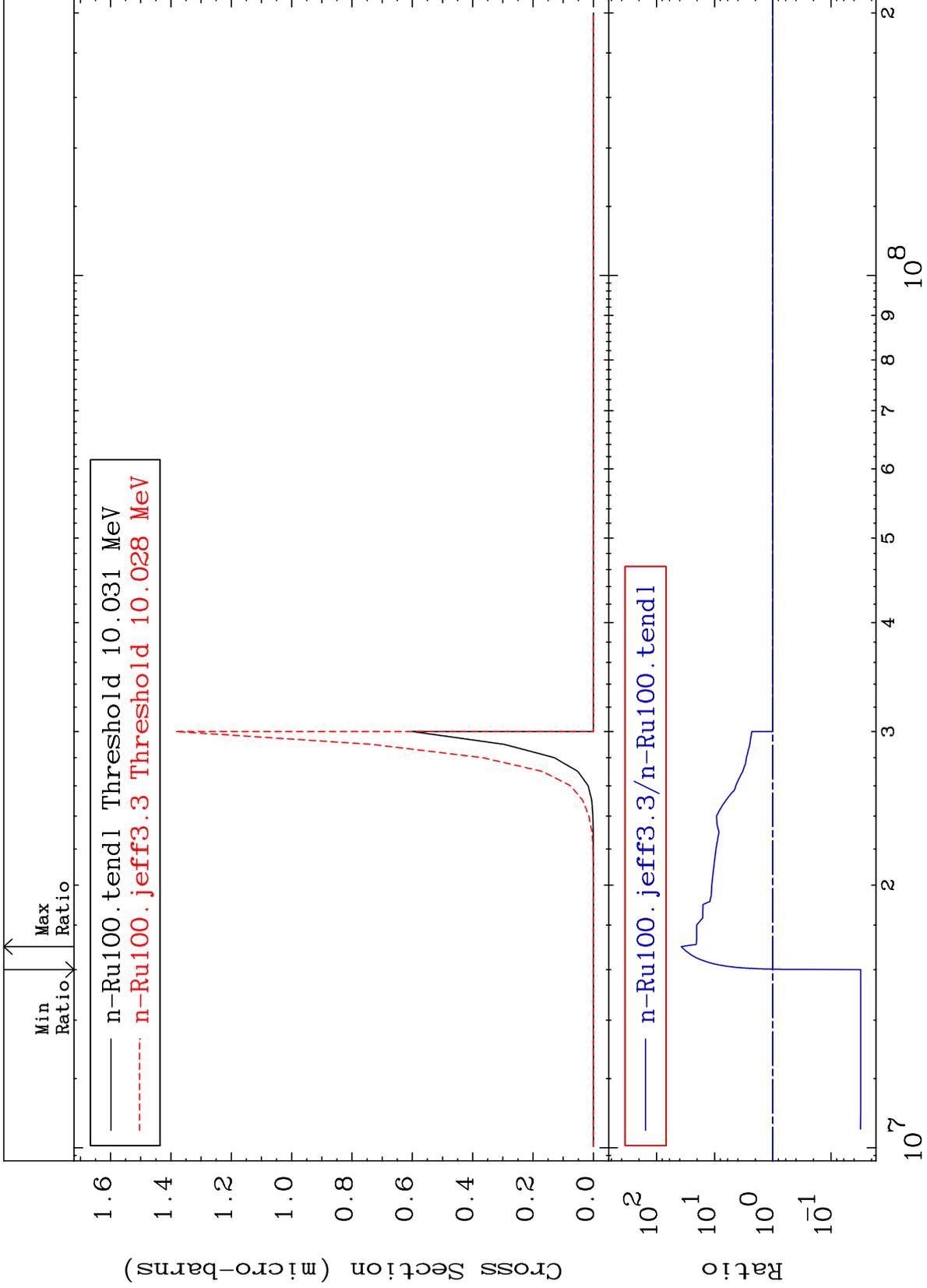
44-Ru-100

MAT 4437

(n, d)  $\alpha$ : 41-Nb-95g

44-Ru-100

Radionuclide Production Cross Section -96.92 To 3660. %

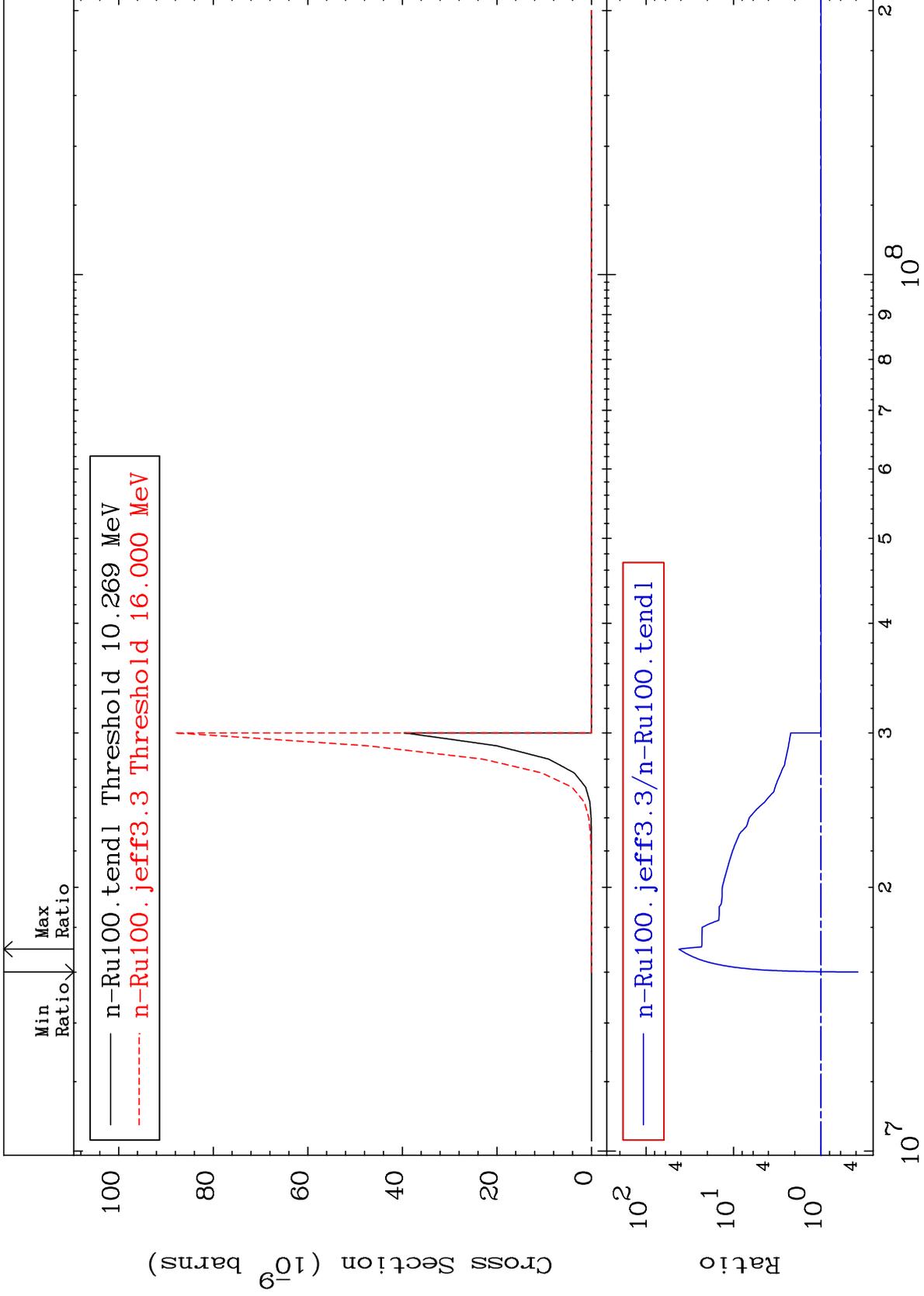


MAT 4437

(n, d)  $\alpha$ : 41-Nb-95m1

44-Ru-100

Radionuclide Production Cross Section -62.02 To 4106. %



90

44-Ru-100

44-Ru-100