

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

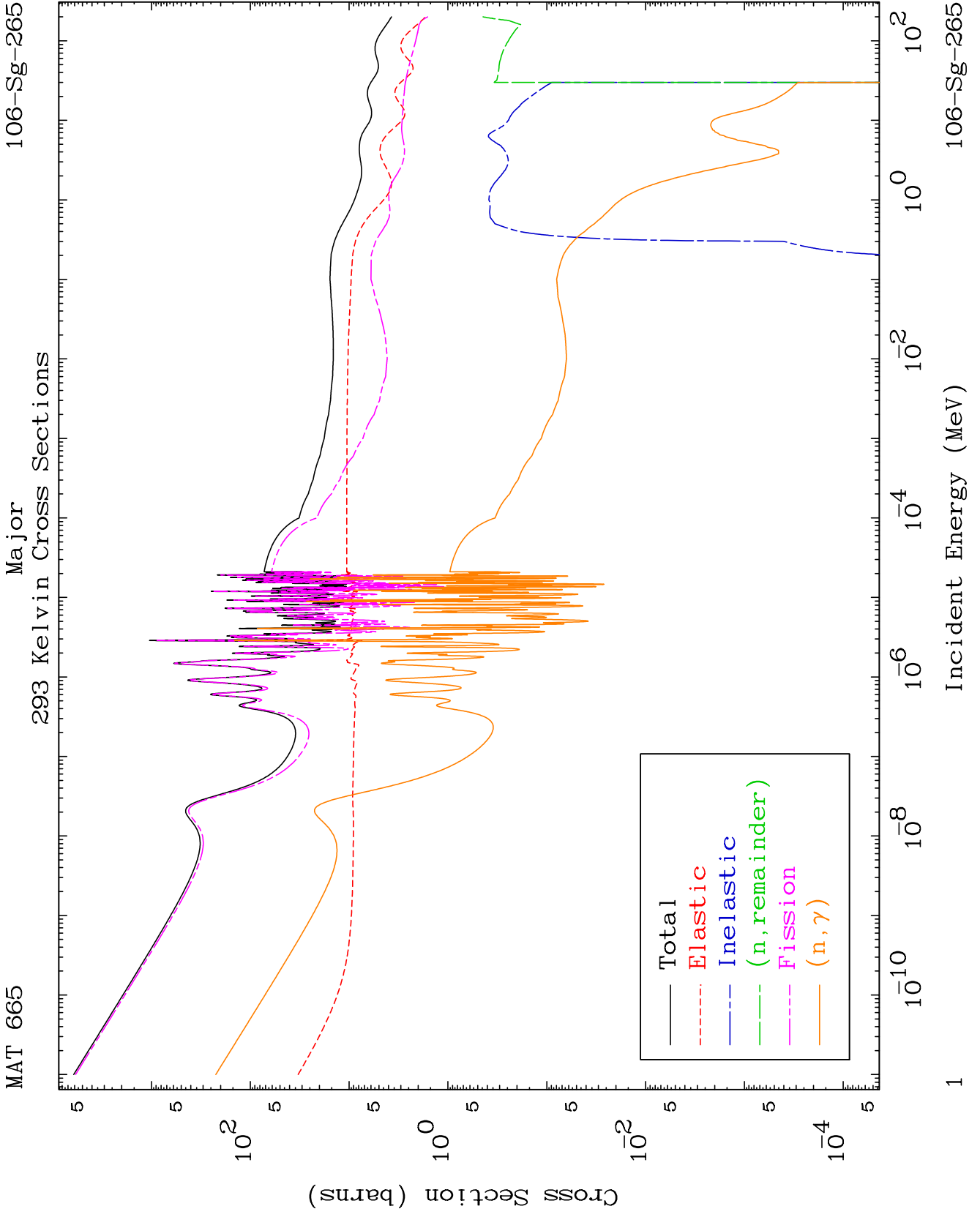
Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

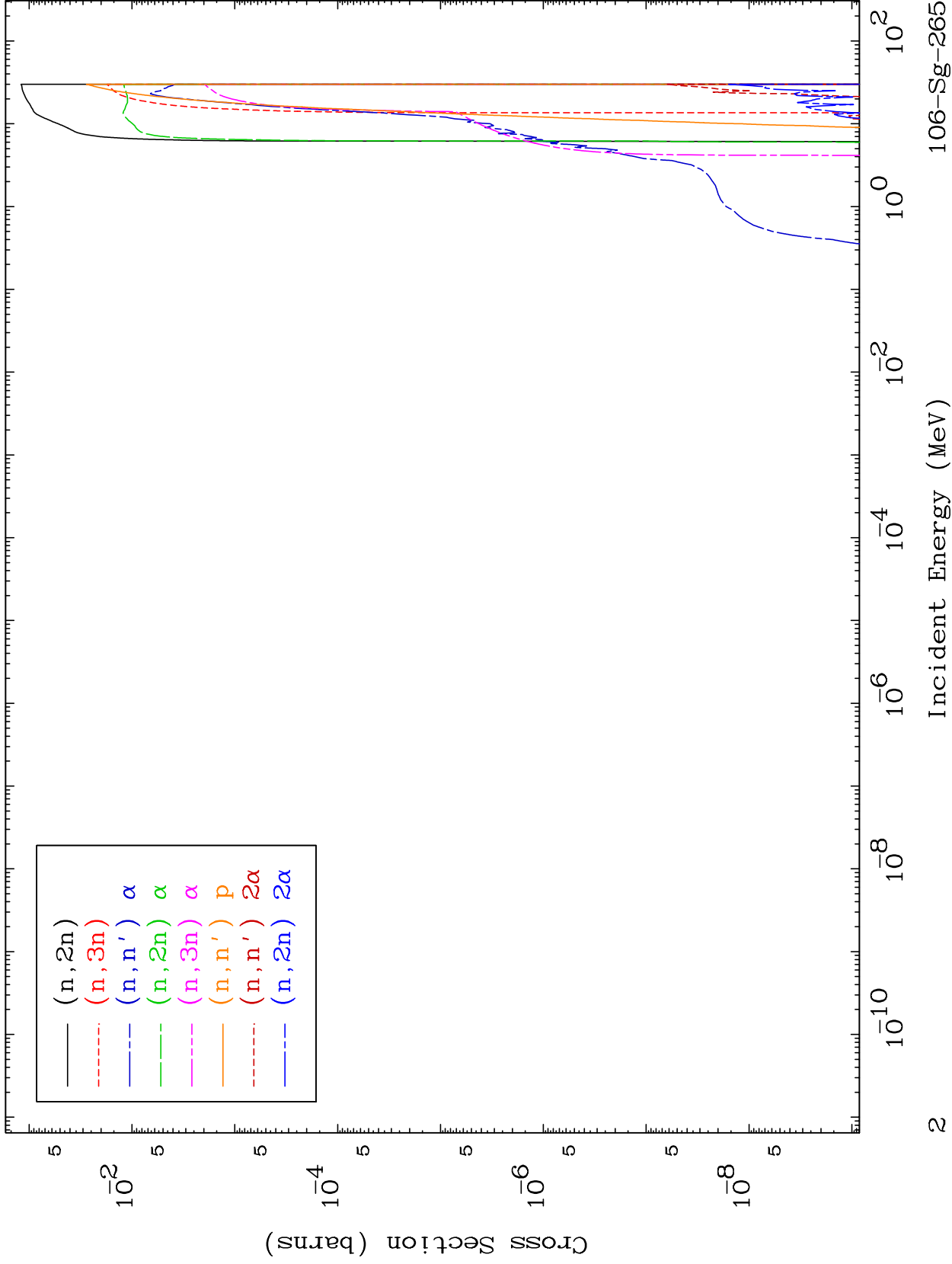
Press Mouse Button to Start

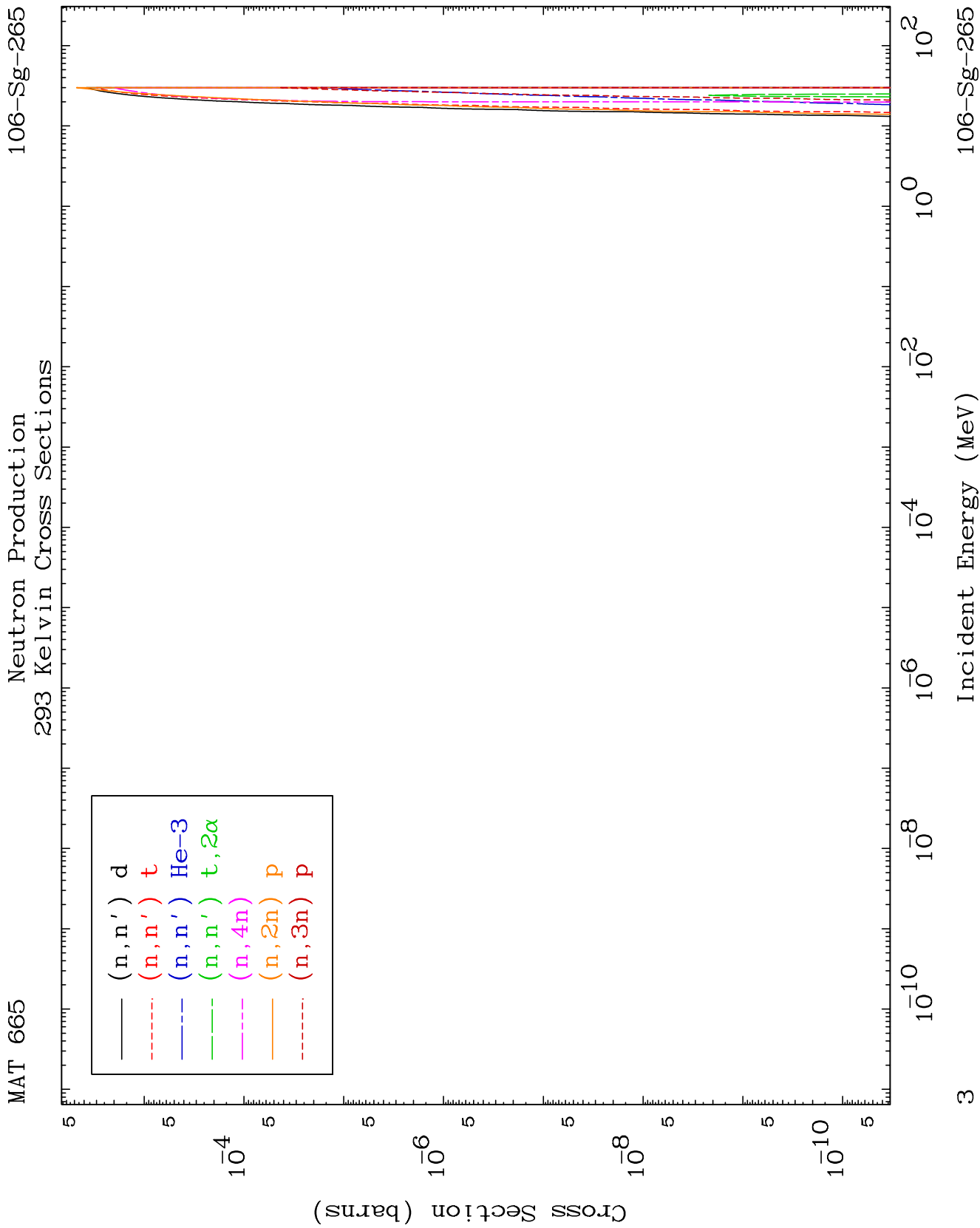


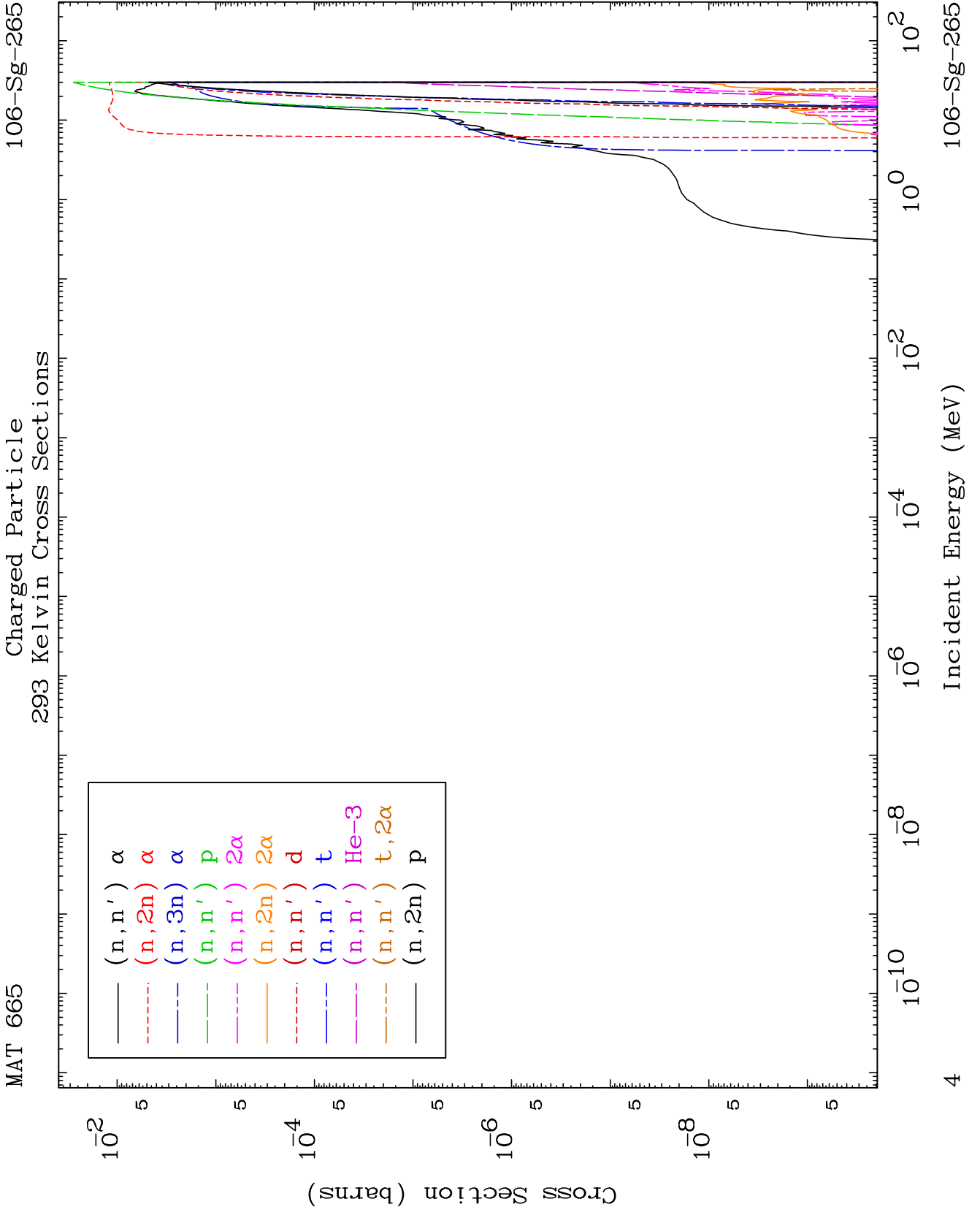
MAT 665

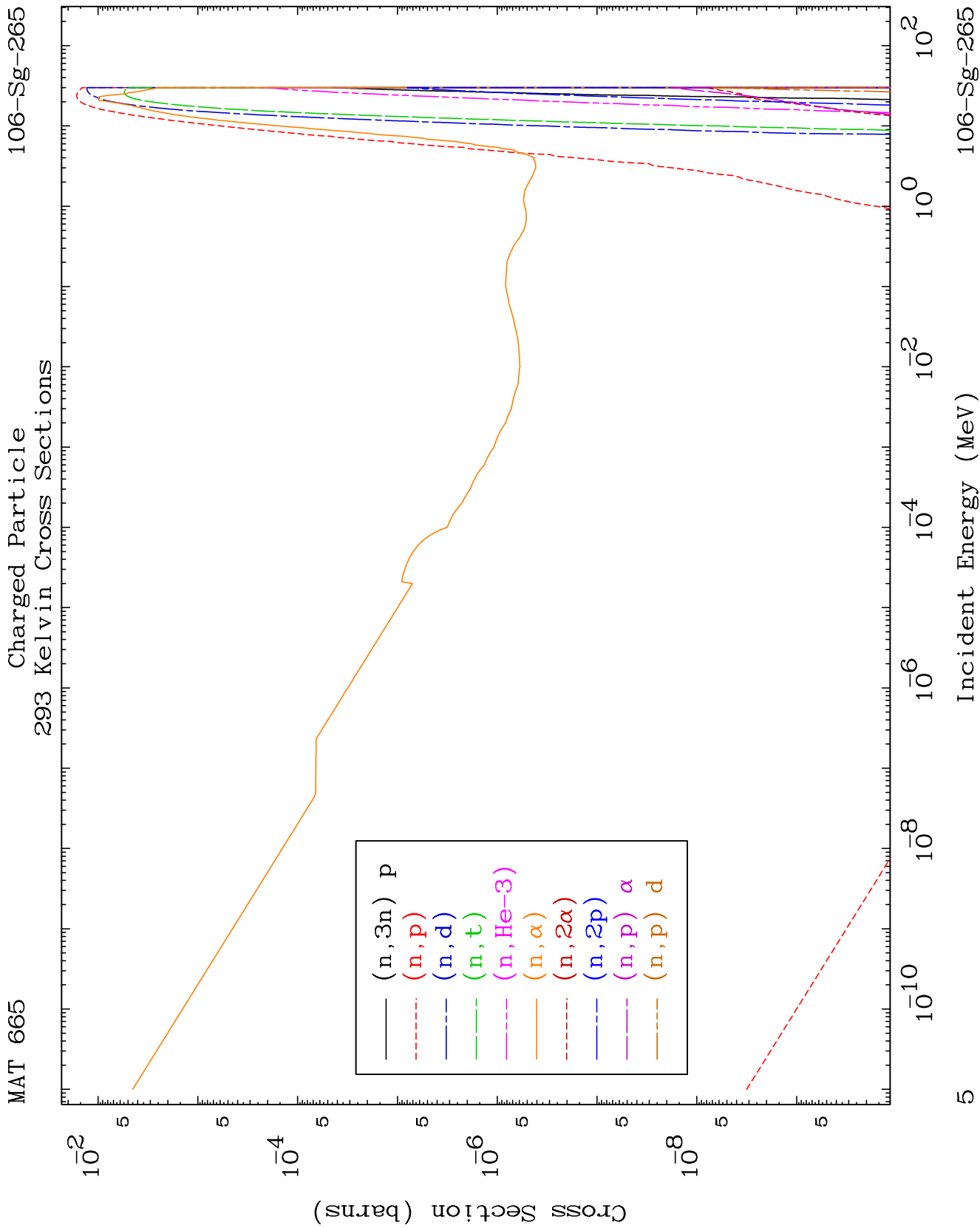
Neutron Production  
293 Kelvin Cross Sections

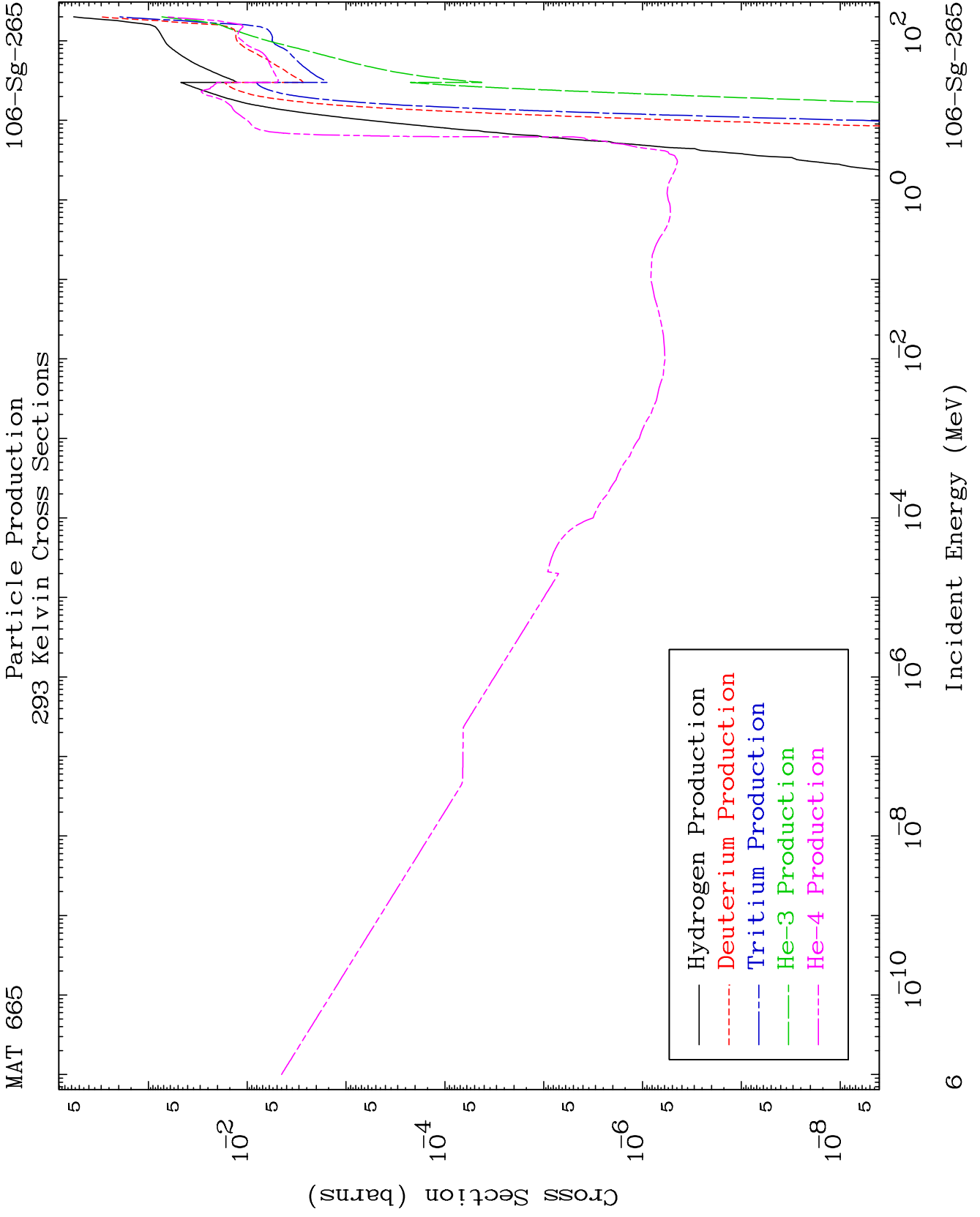
106-Sg-265







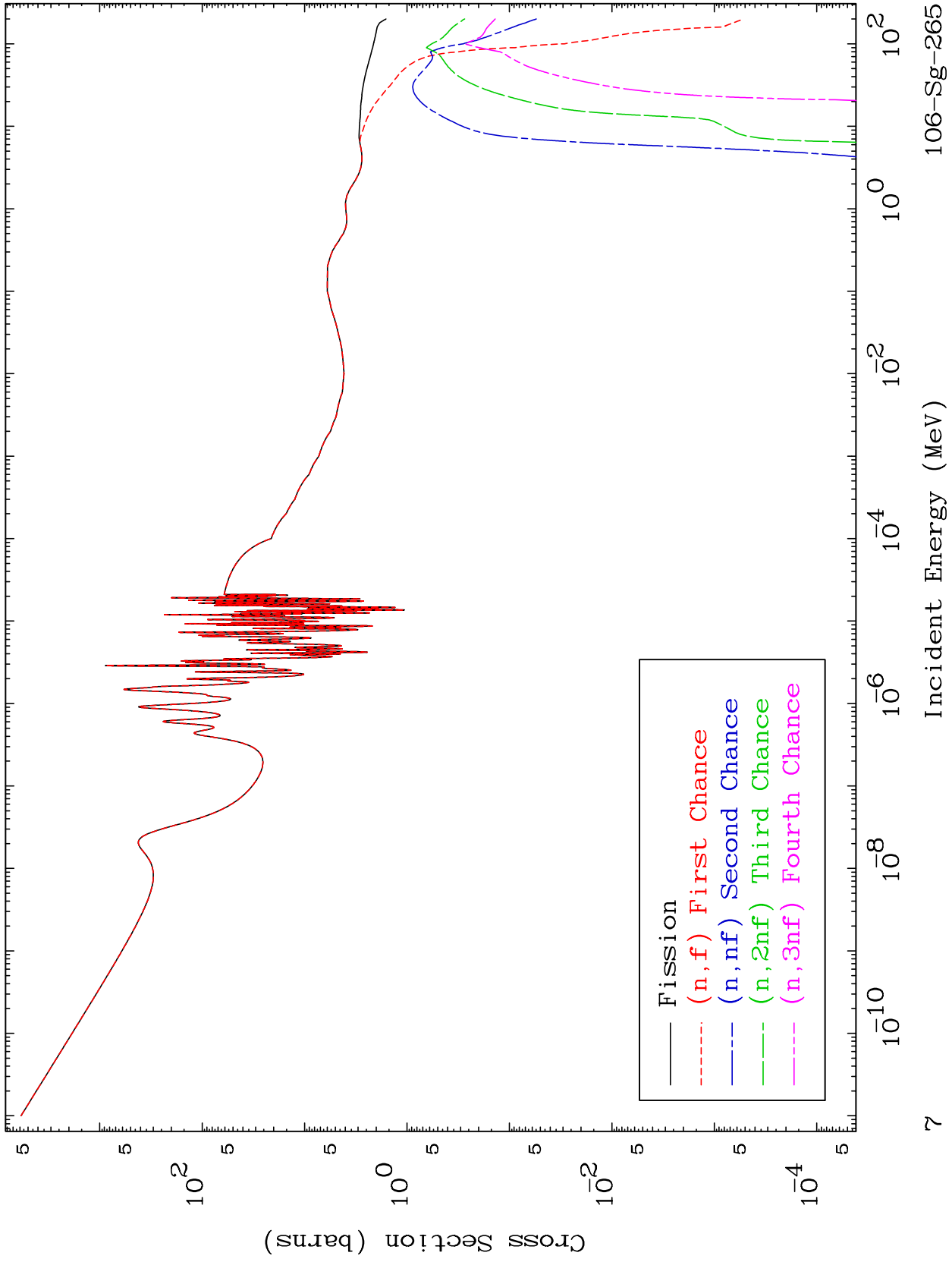




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Fission  
293 Kelvin Cross Sections

106-Sg-265

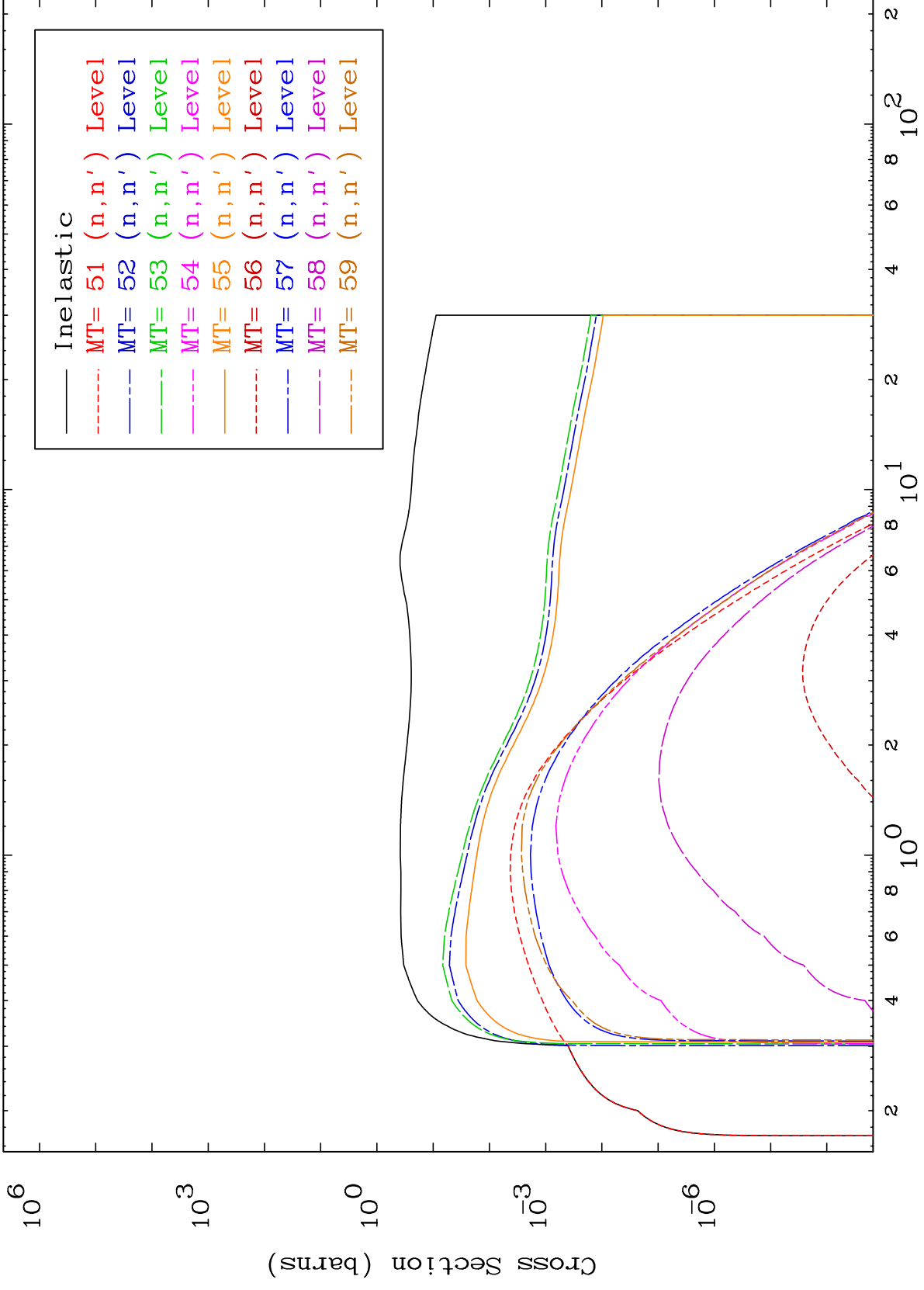


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(n,n') Level

106-Sg-265

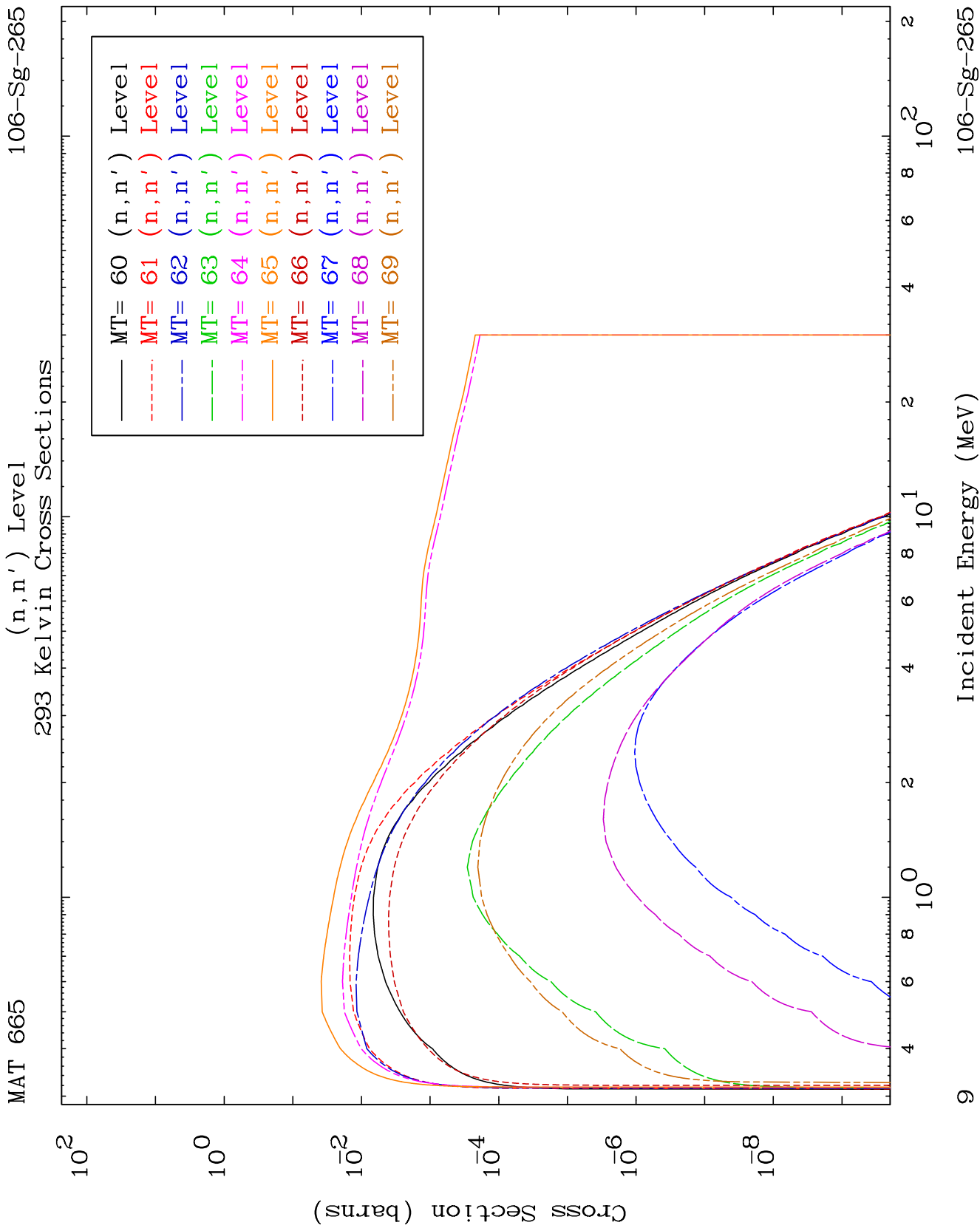
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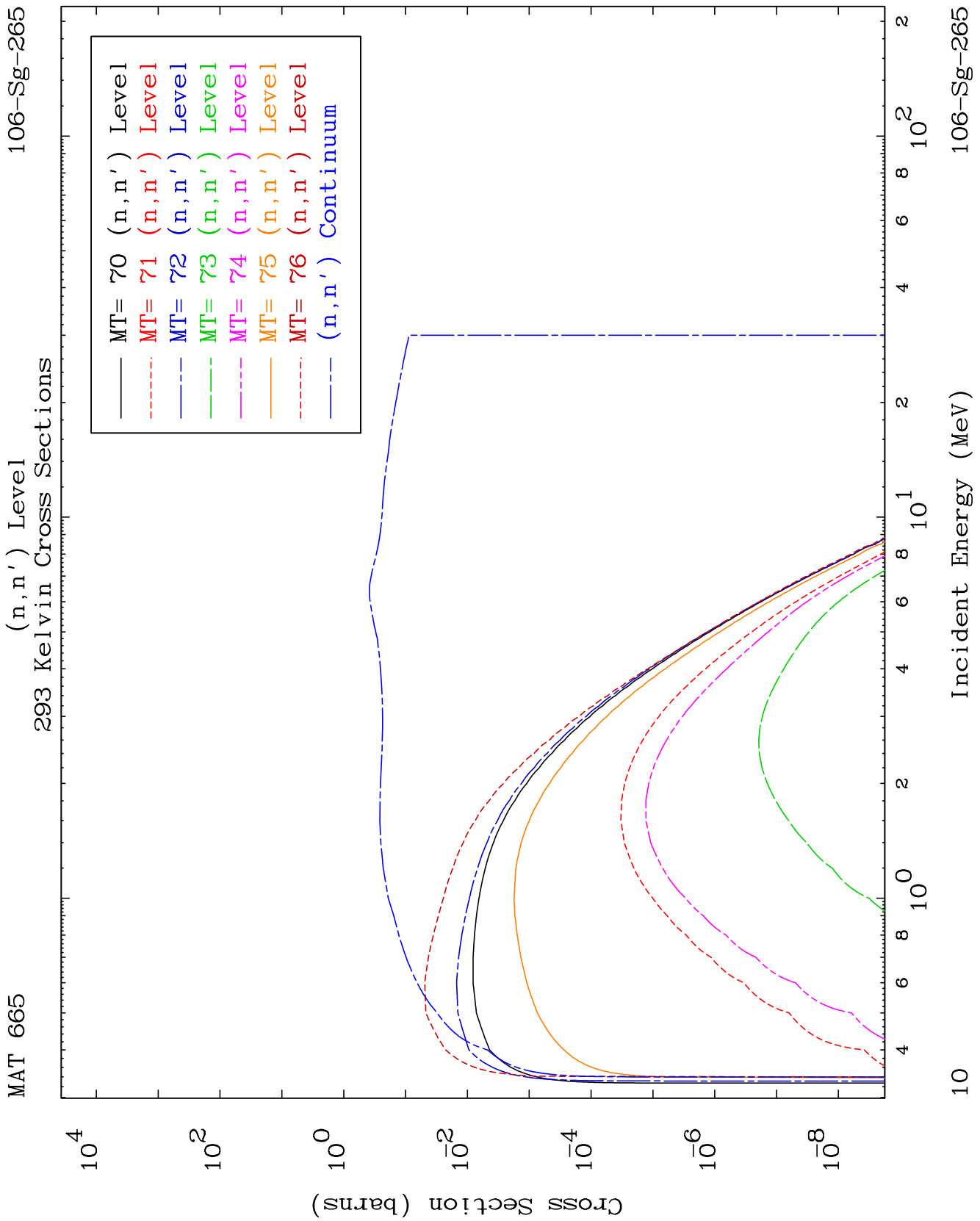


8

Incident Energy (MeV)

106-Sg-265

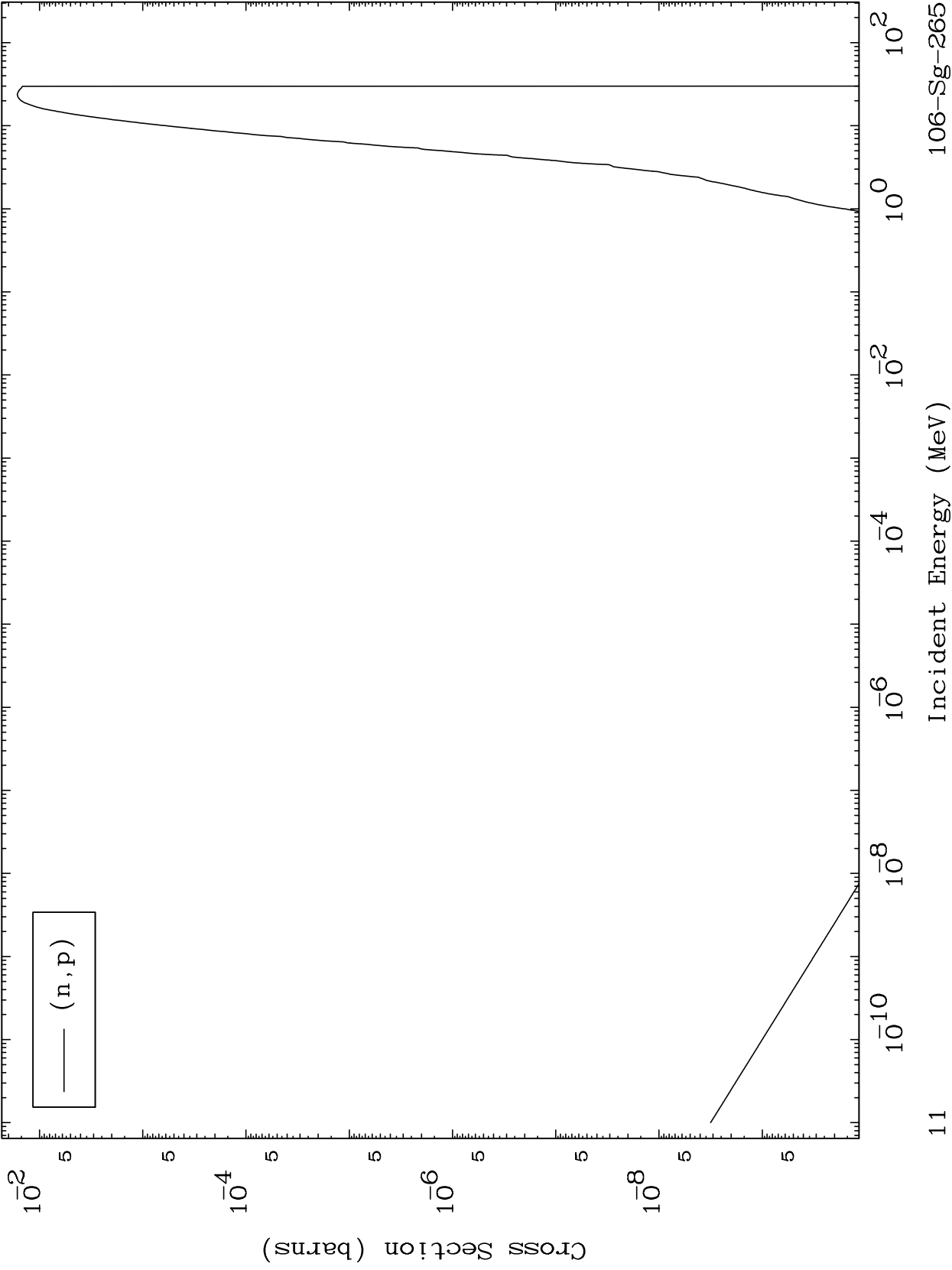




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(n,p) Levels  
293 Kelvin Cross Sections

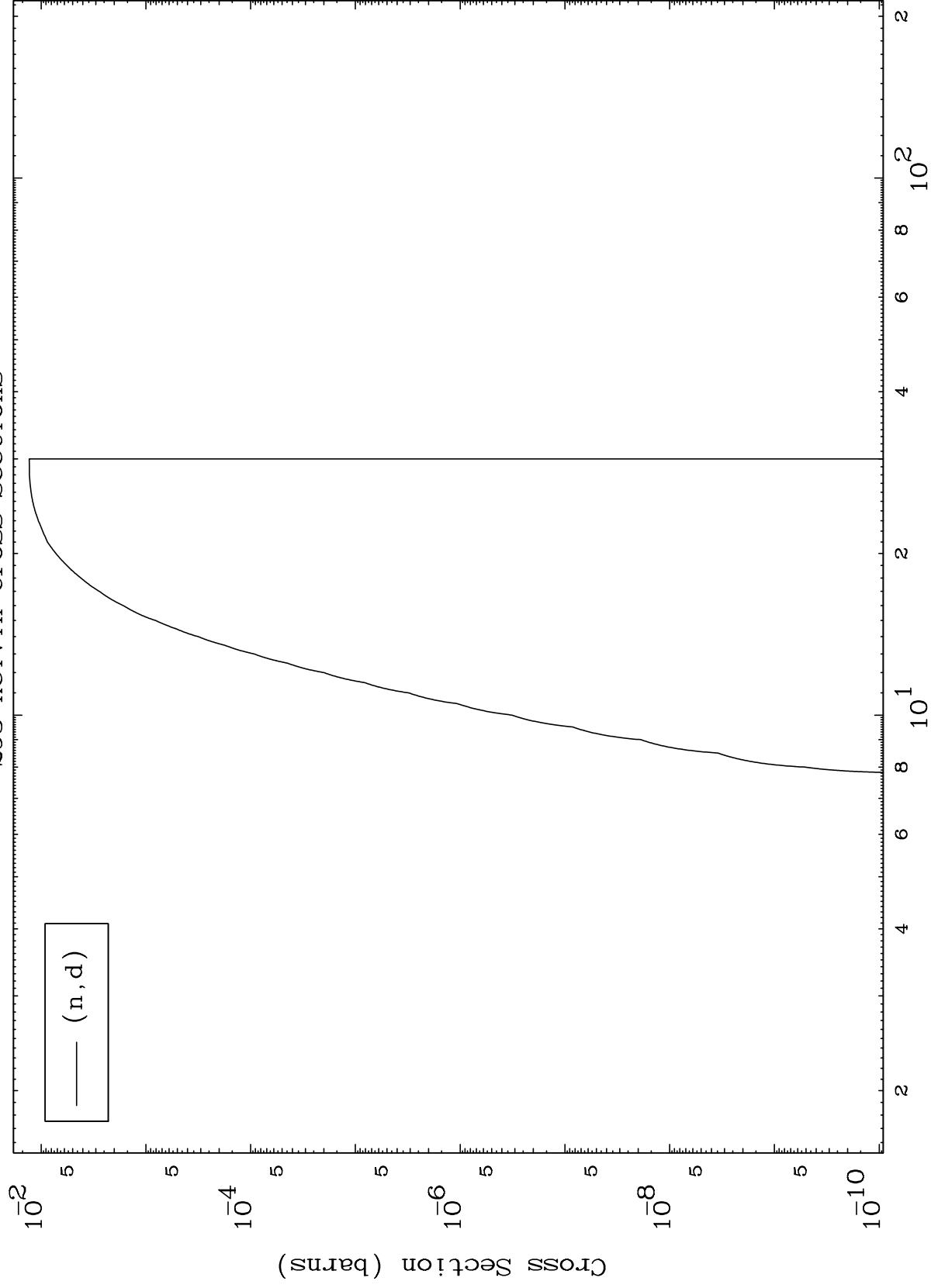
106-Sg-265



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(n,d) Levels  
293 Kelvin Cross Sections

106-Sg-265



12

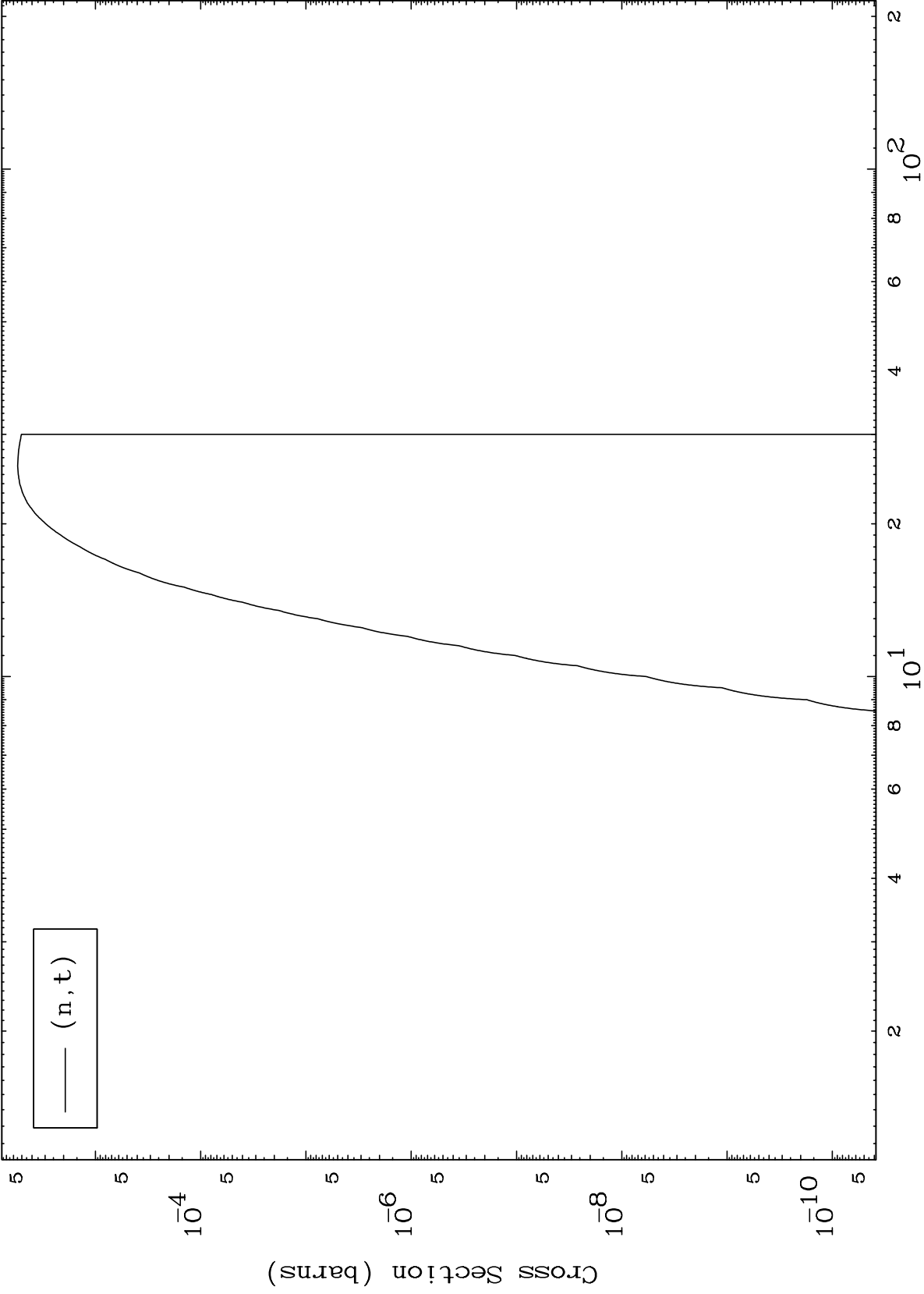
Incident Energy (MeV)

106-Sg-265

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(n,t) Levels  
293 Kelvin Cross Sections

106-Sg-265



13

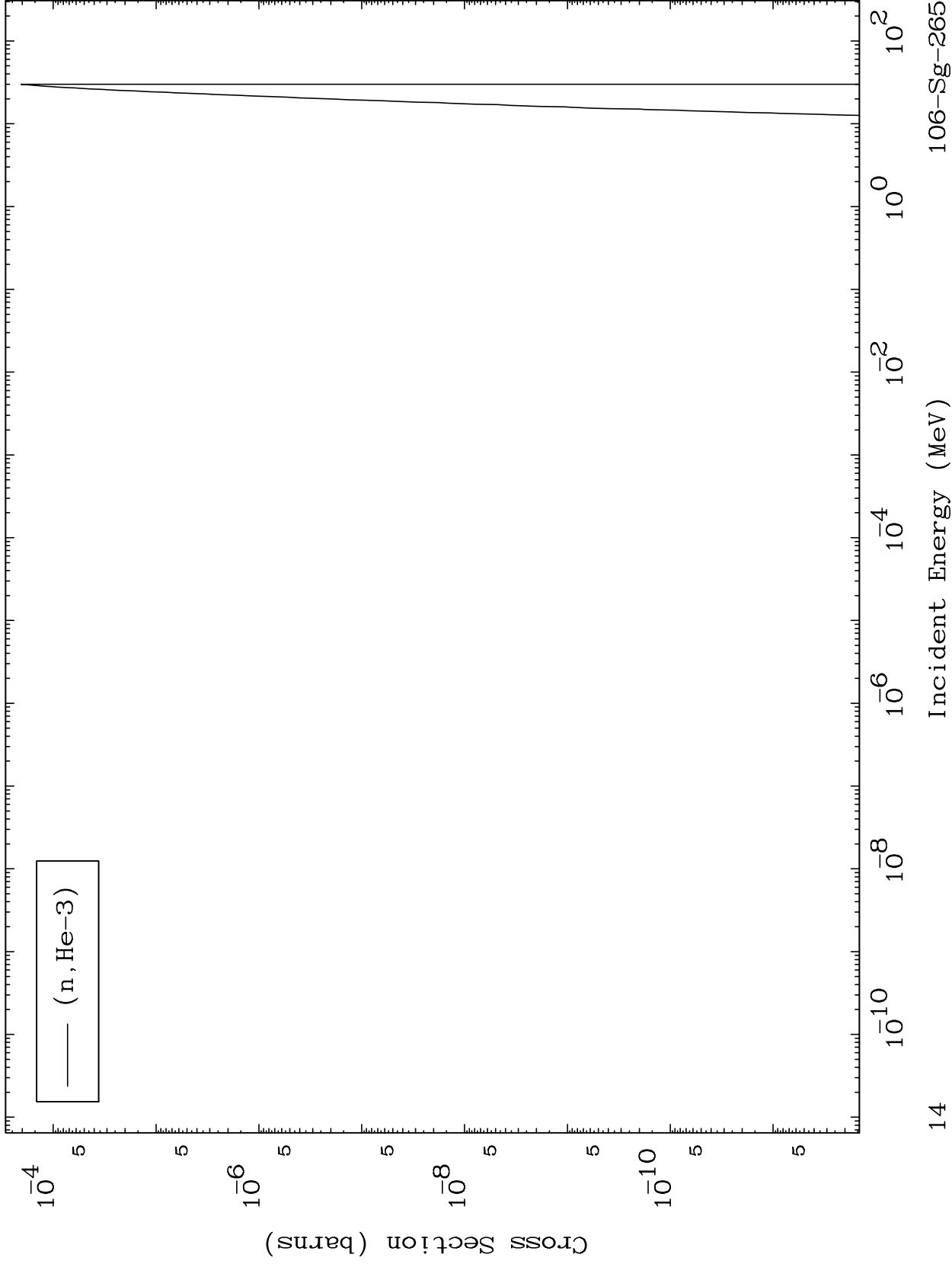
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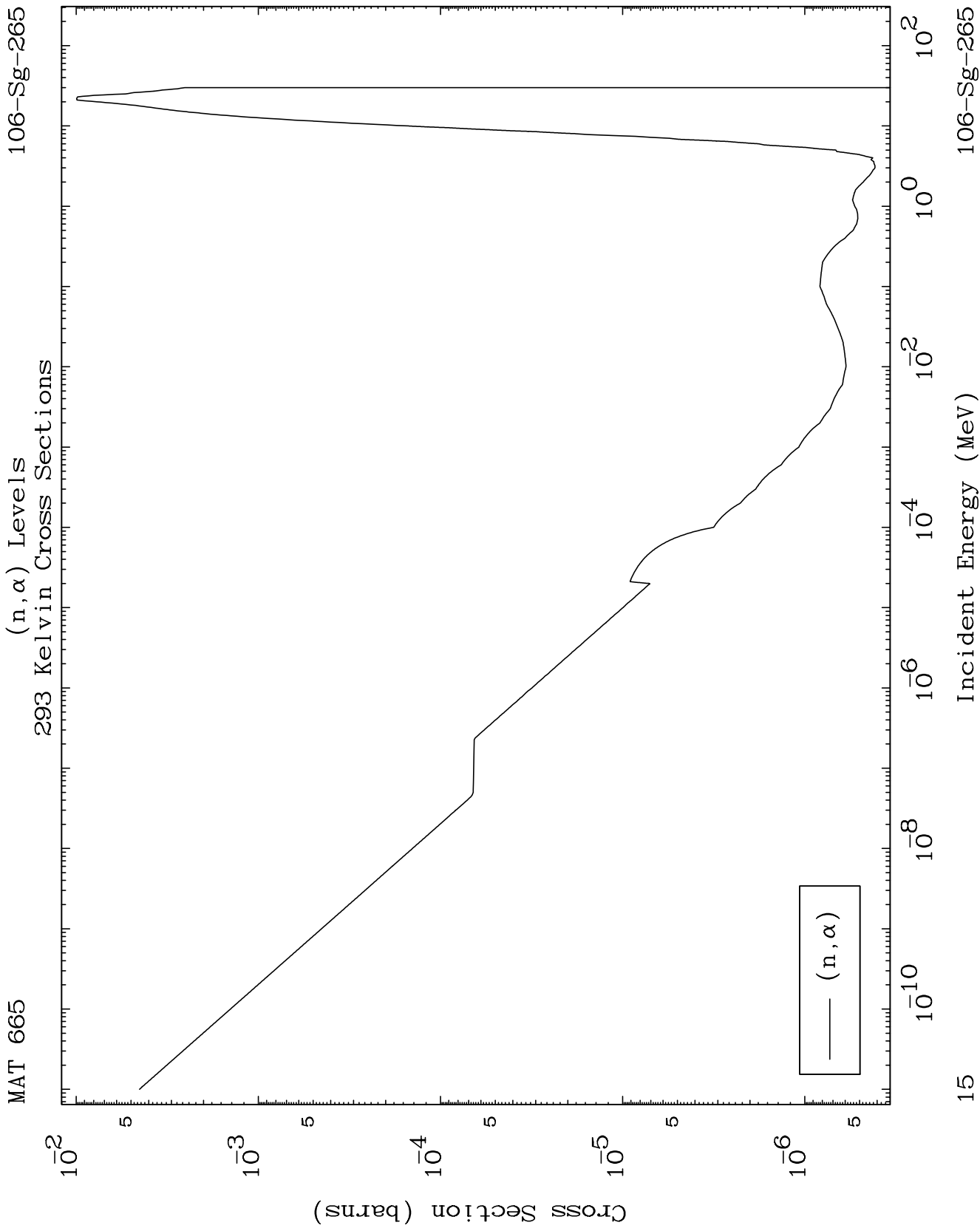
106-Sg-265

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(n,He3) Levels  
293 Kelvin Cross Sections

106-Sg-265

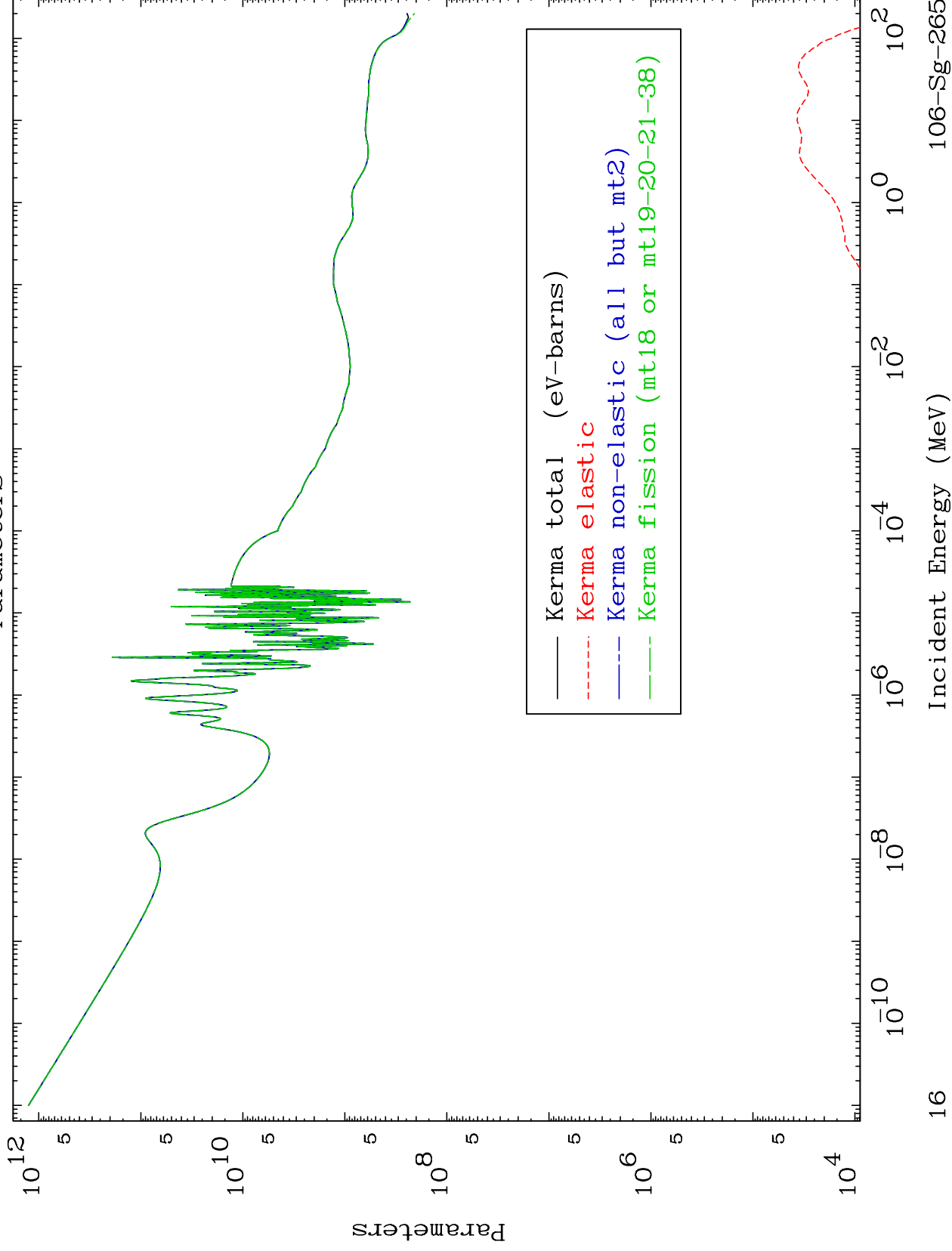




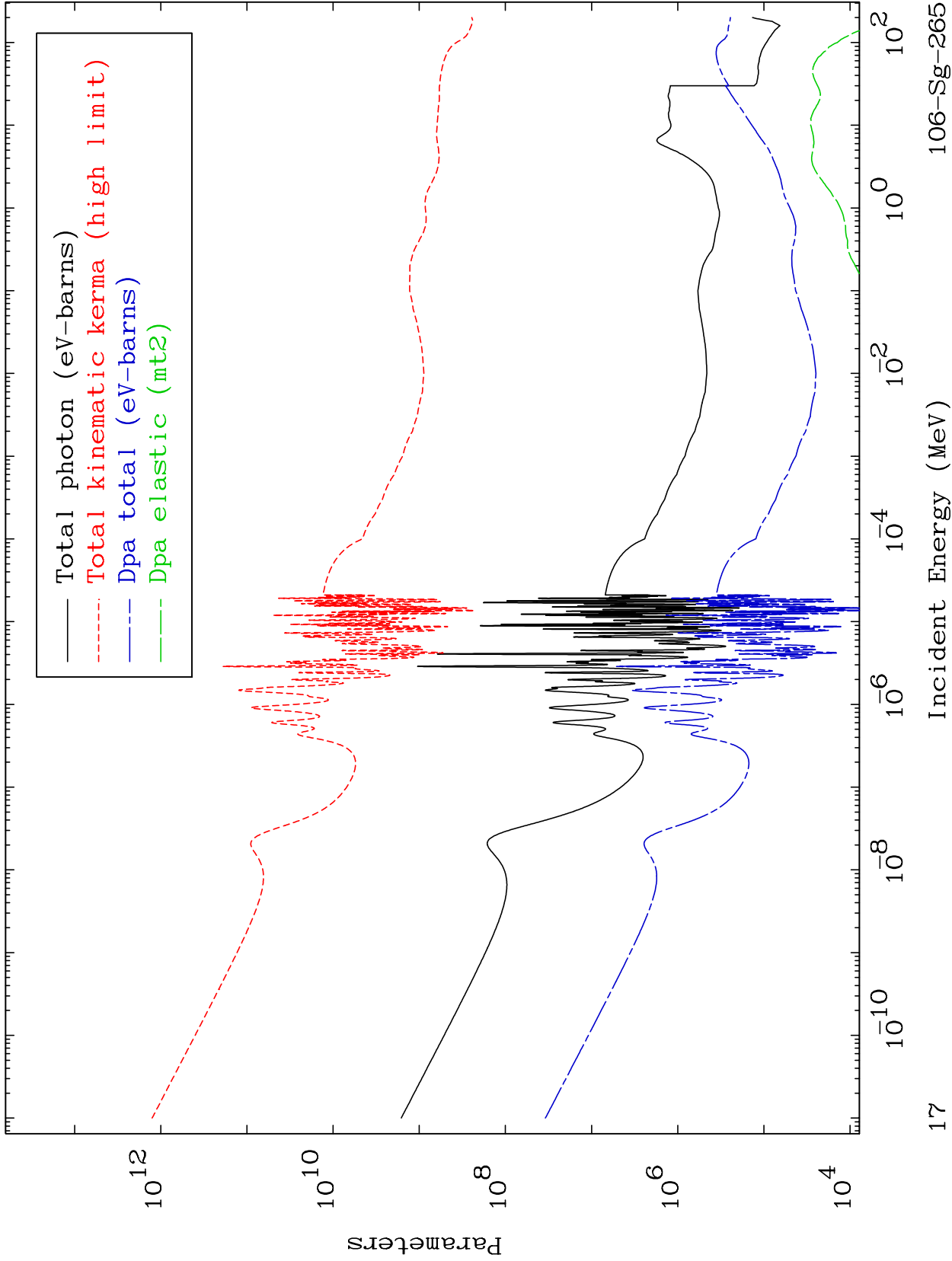
MAT 665

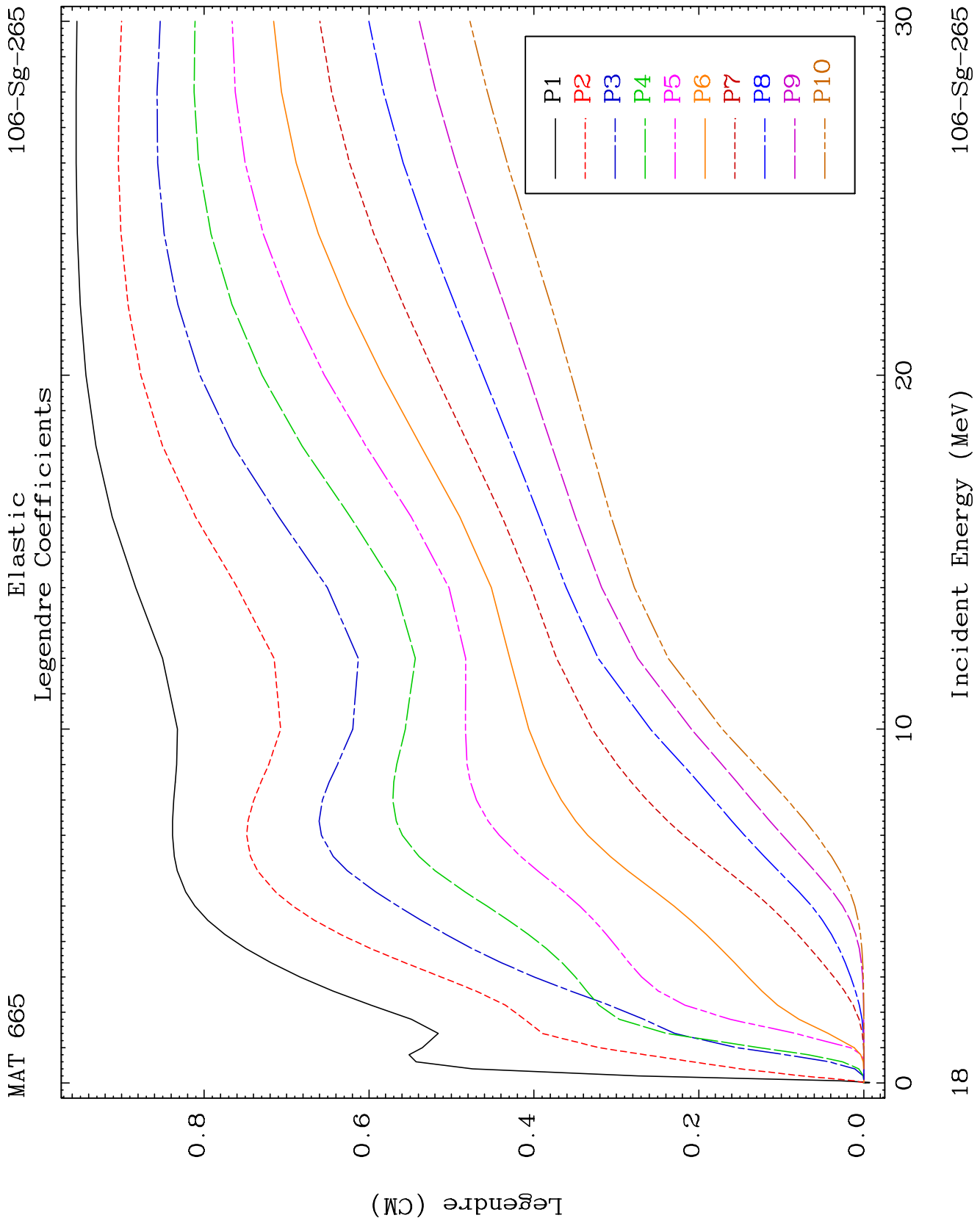
Energy Release  
Parameters

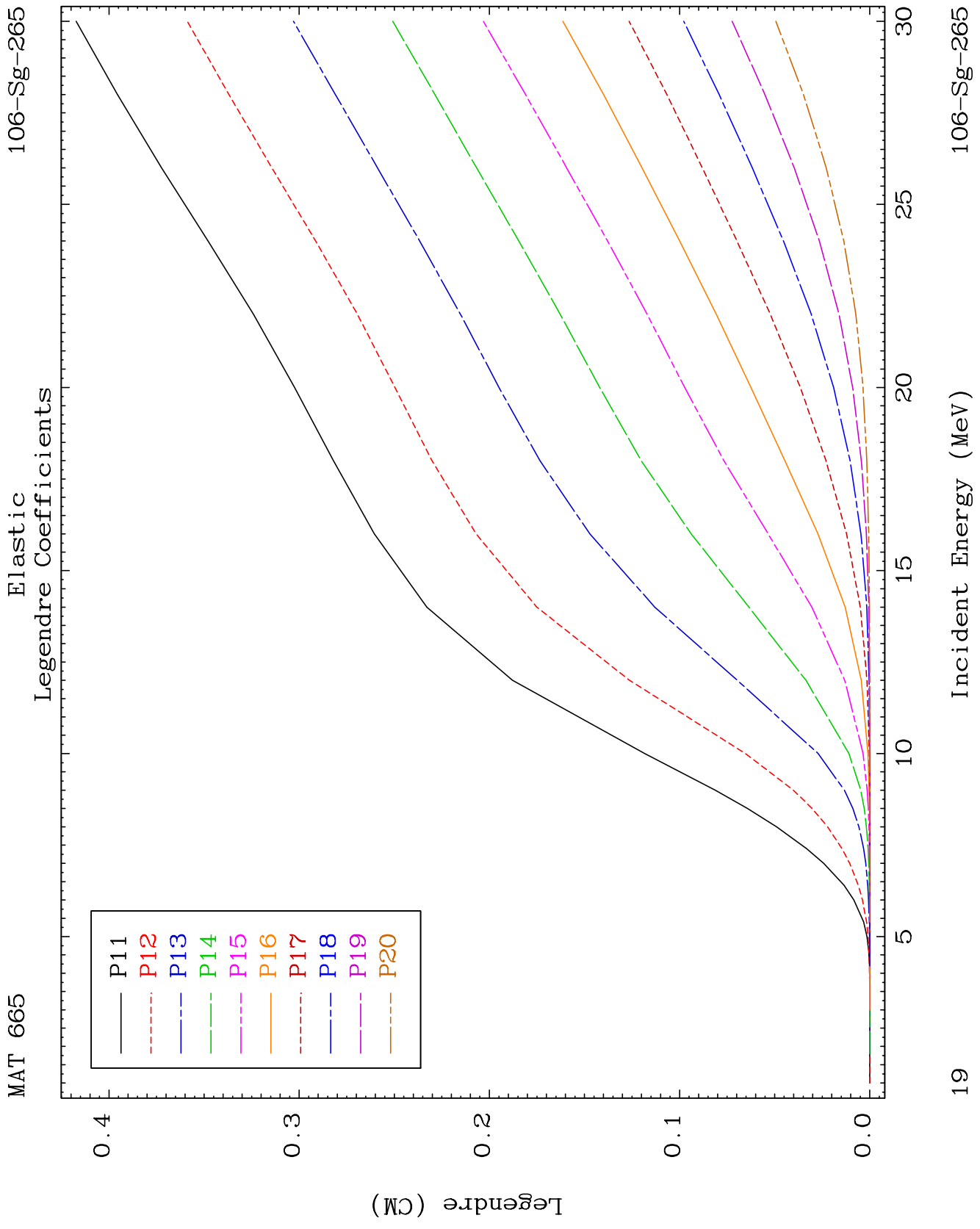
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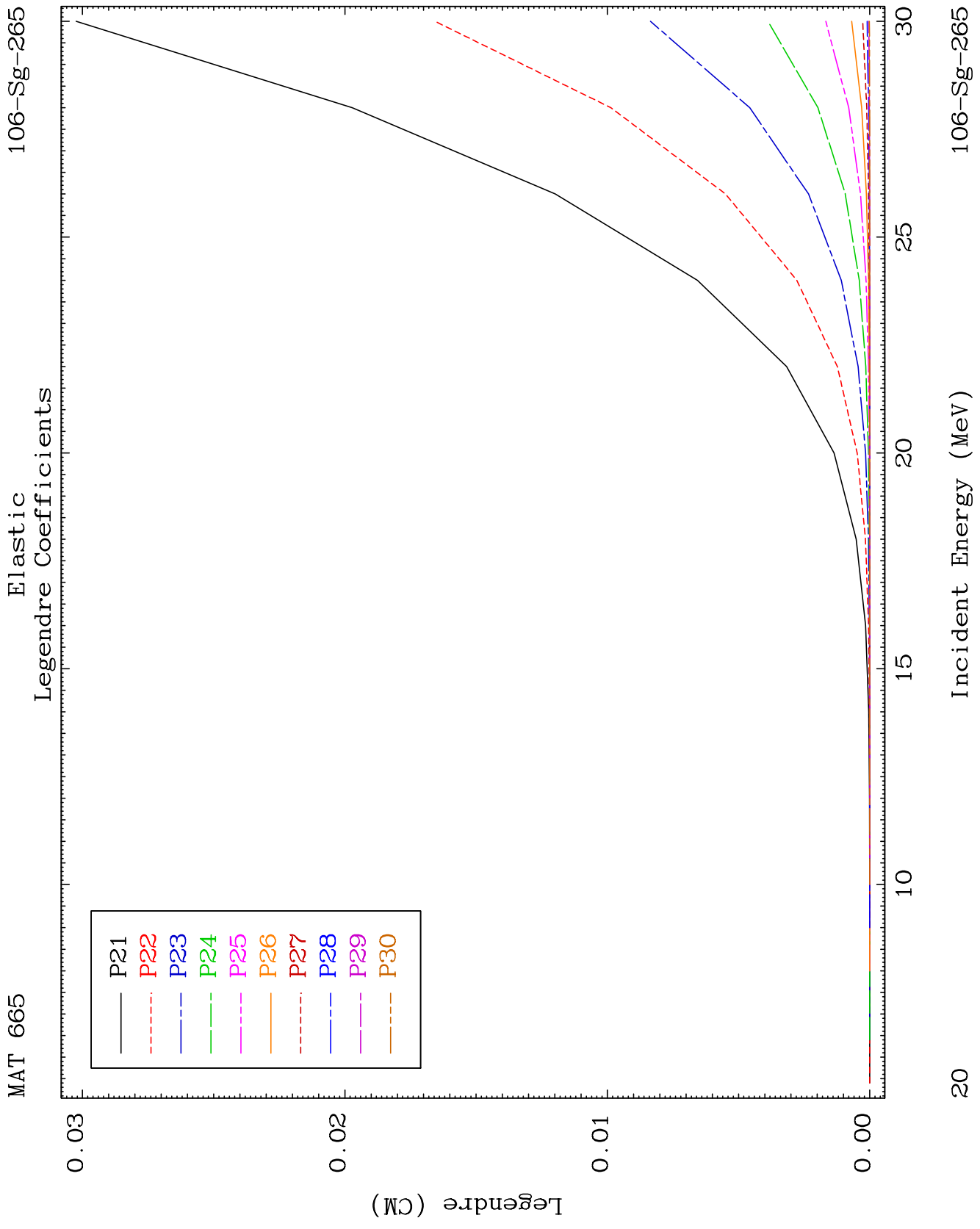


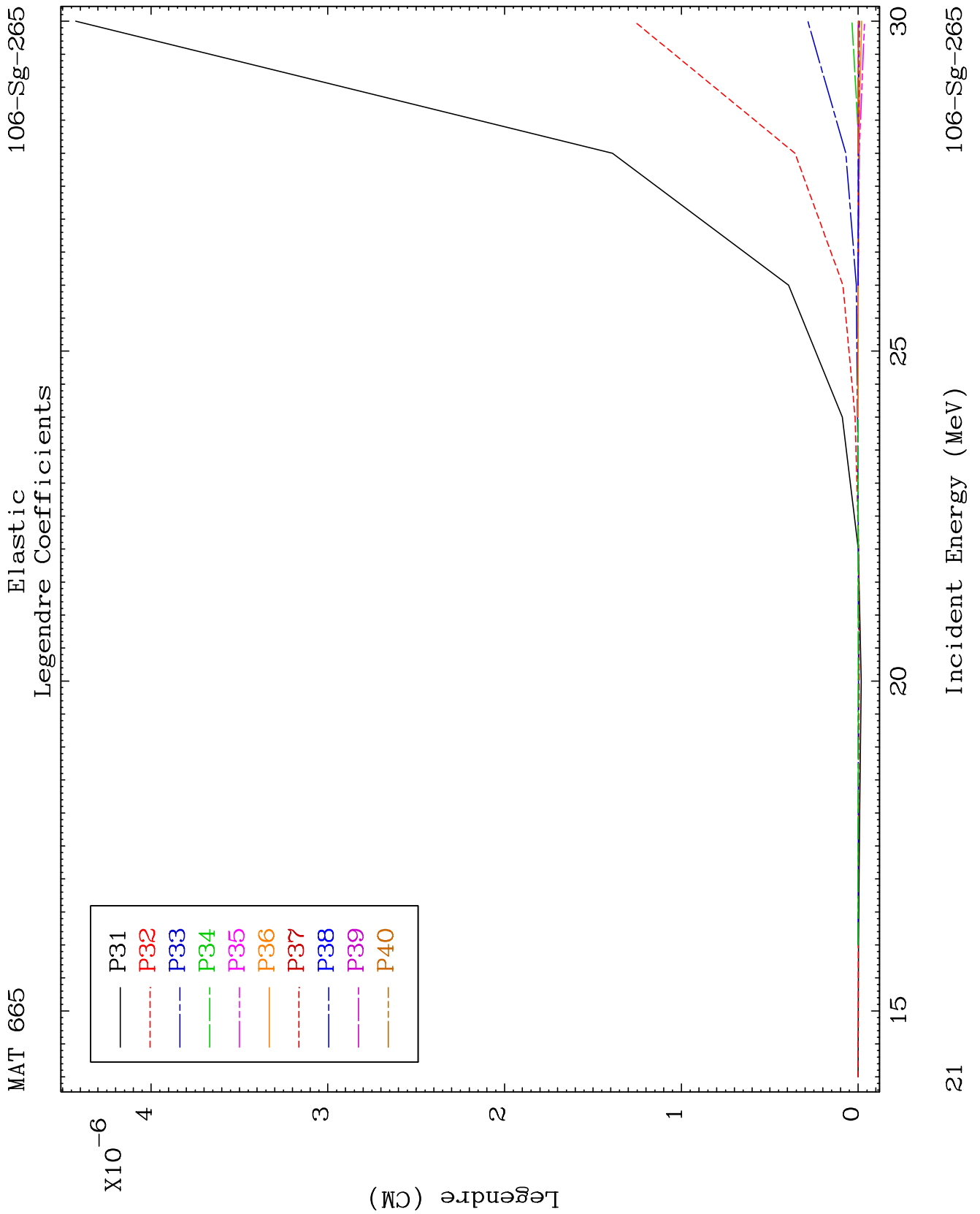
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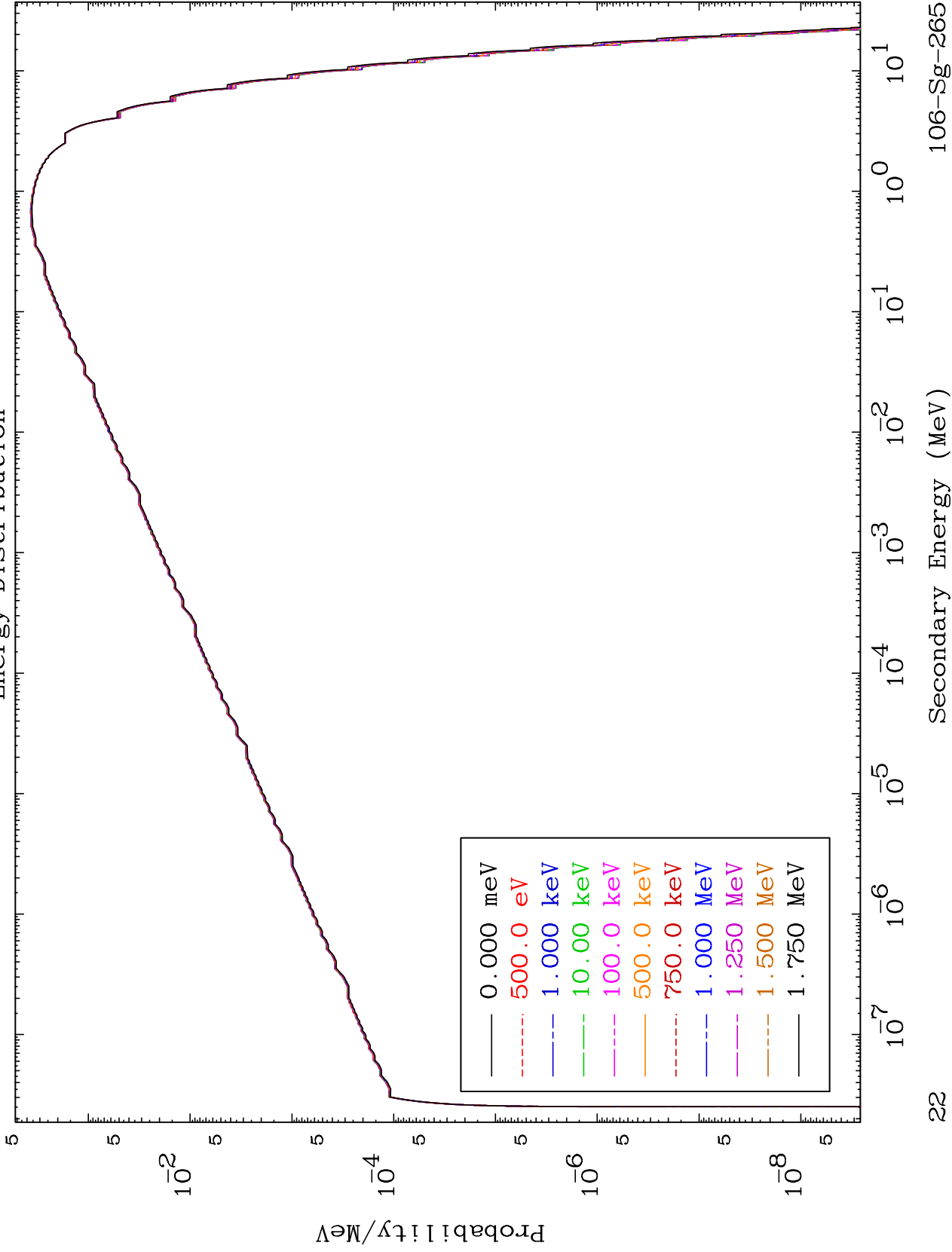


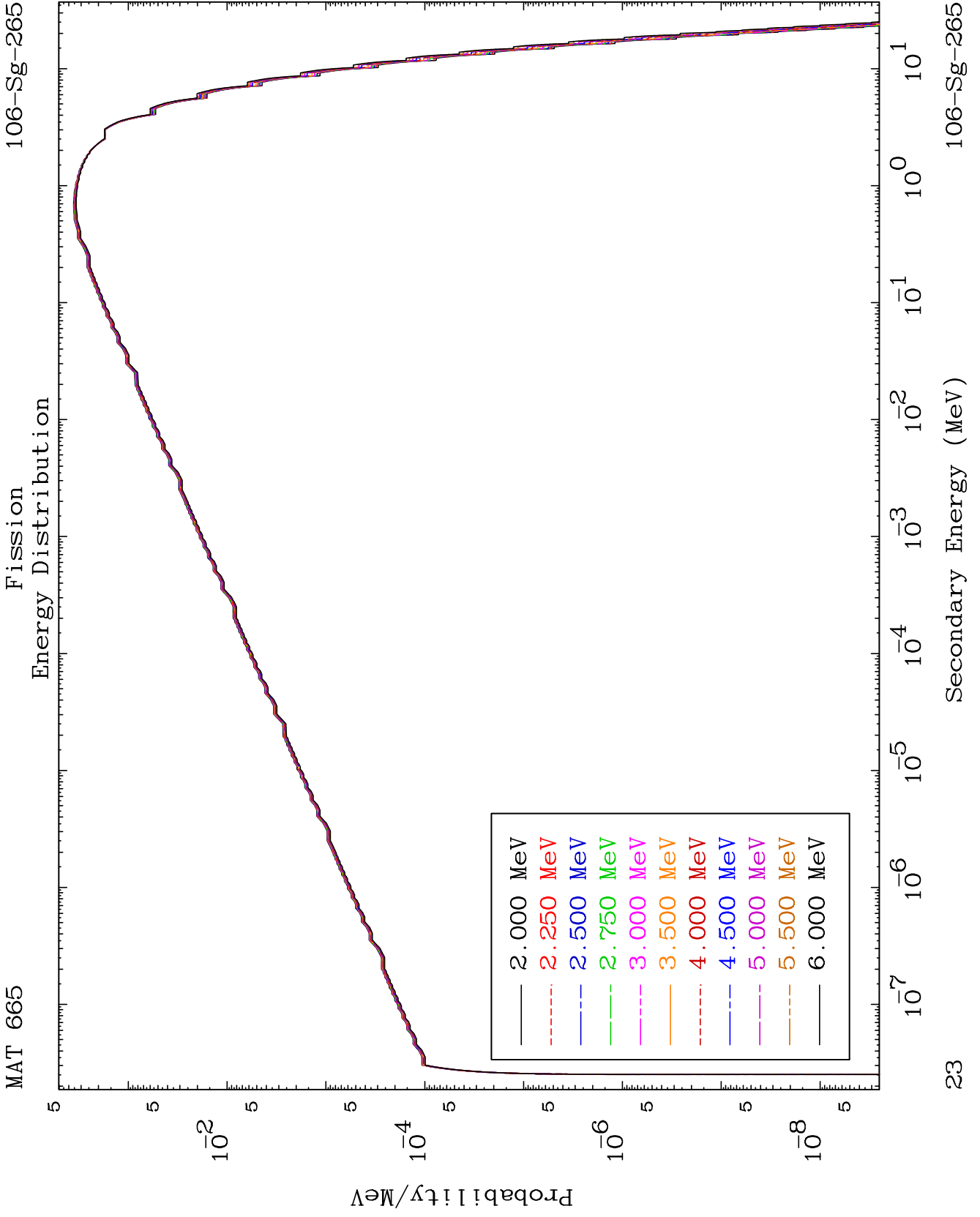


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### Fission Energy Distribution

106-Sg-265

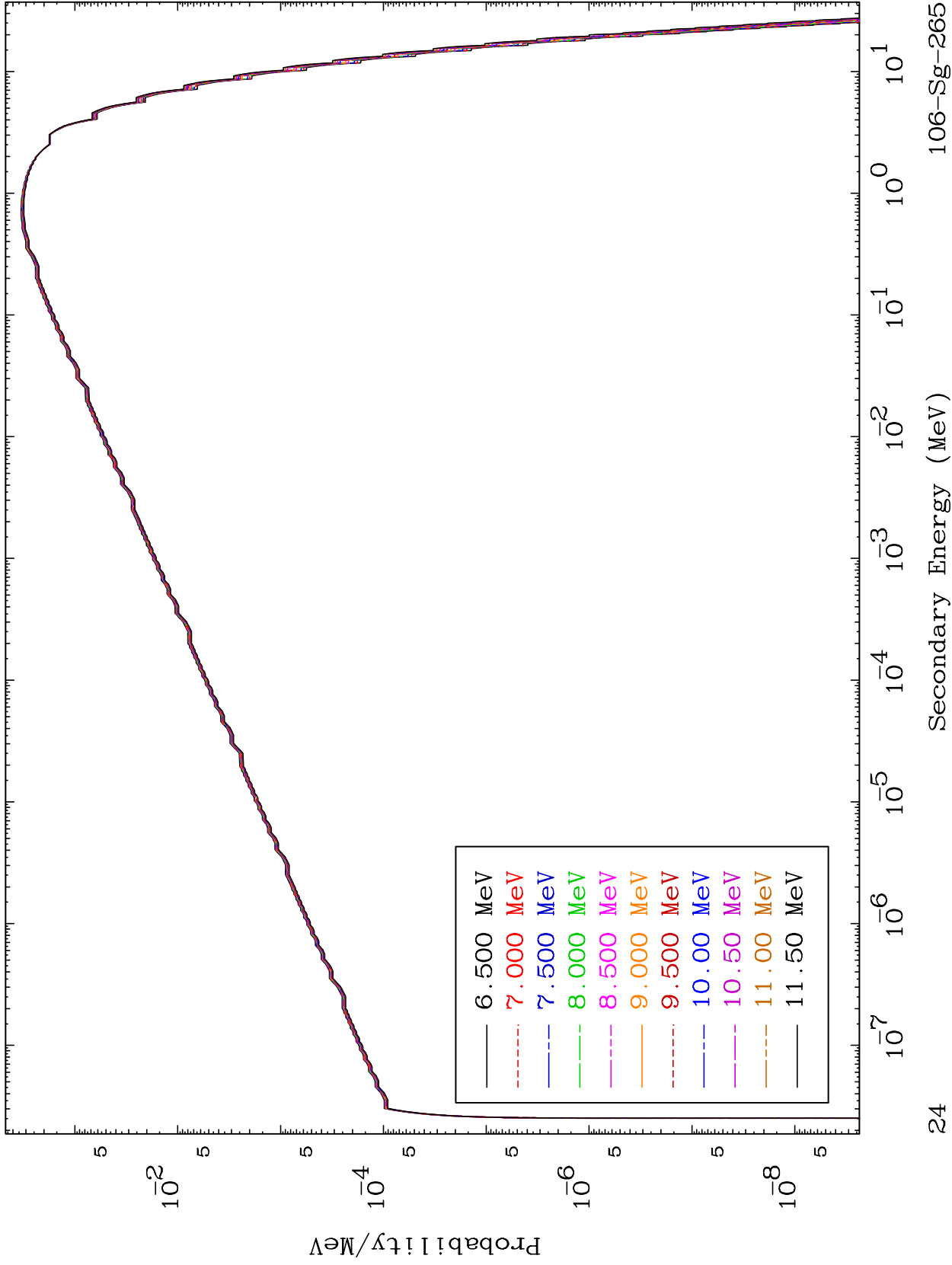




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# Fission Energy Distribution

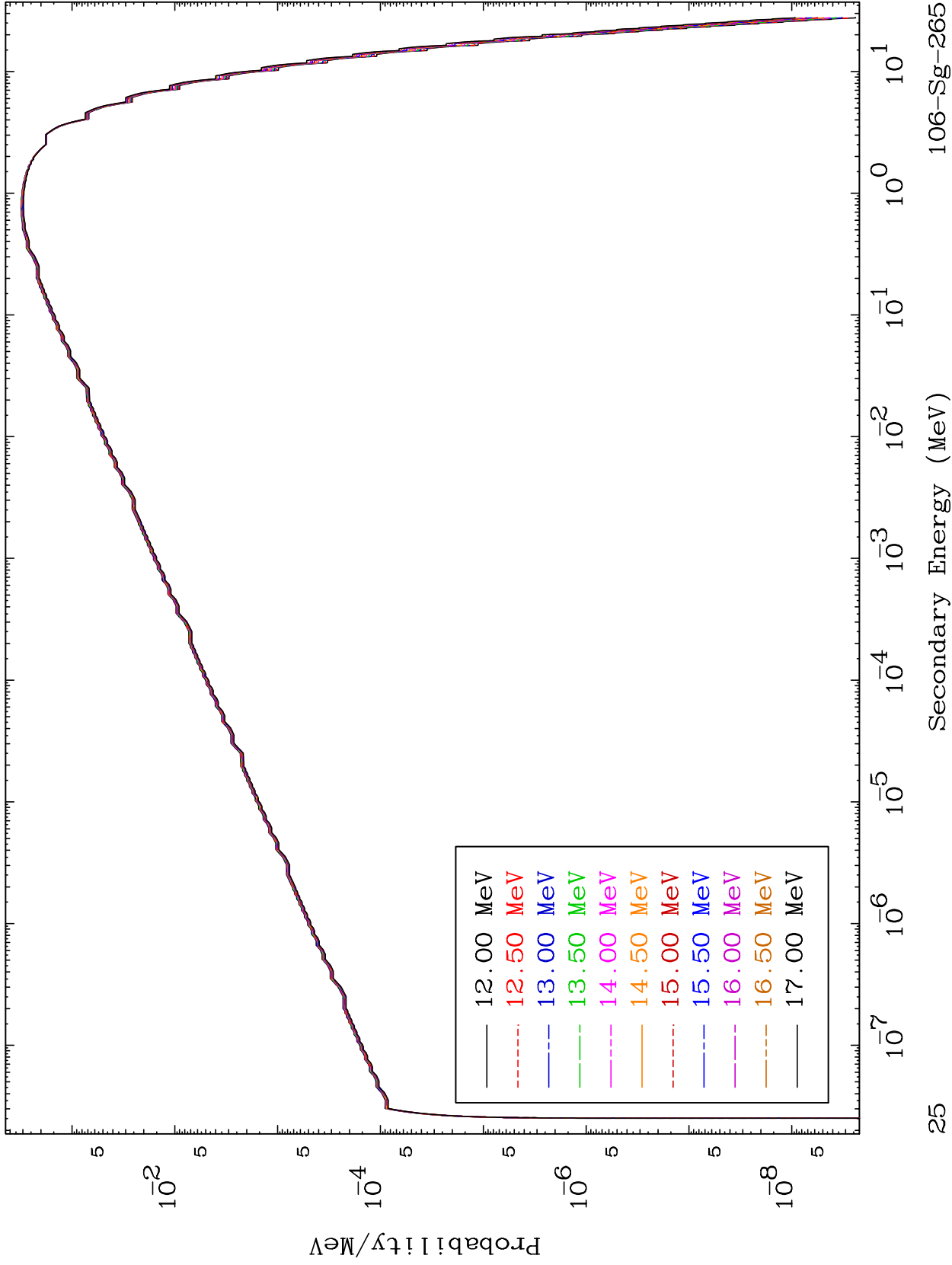
106-Sg-265



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### Fission Energy Distribution

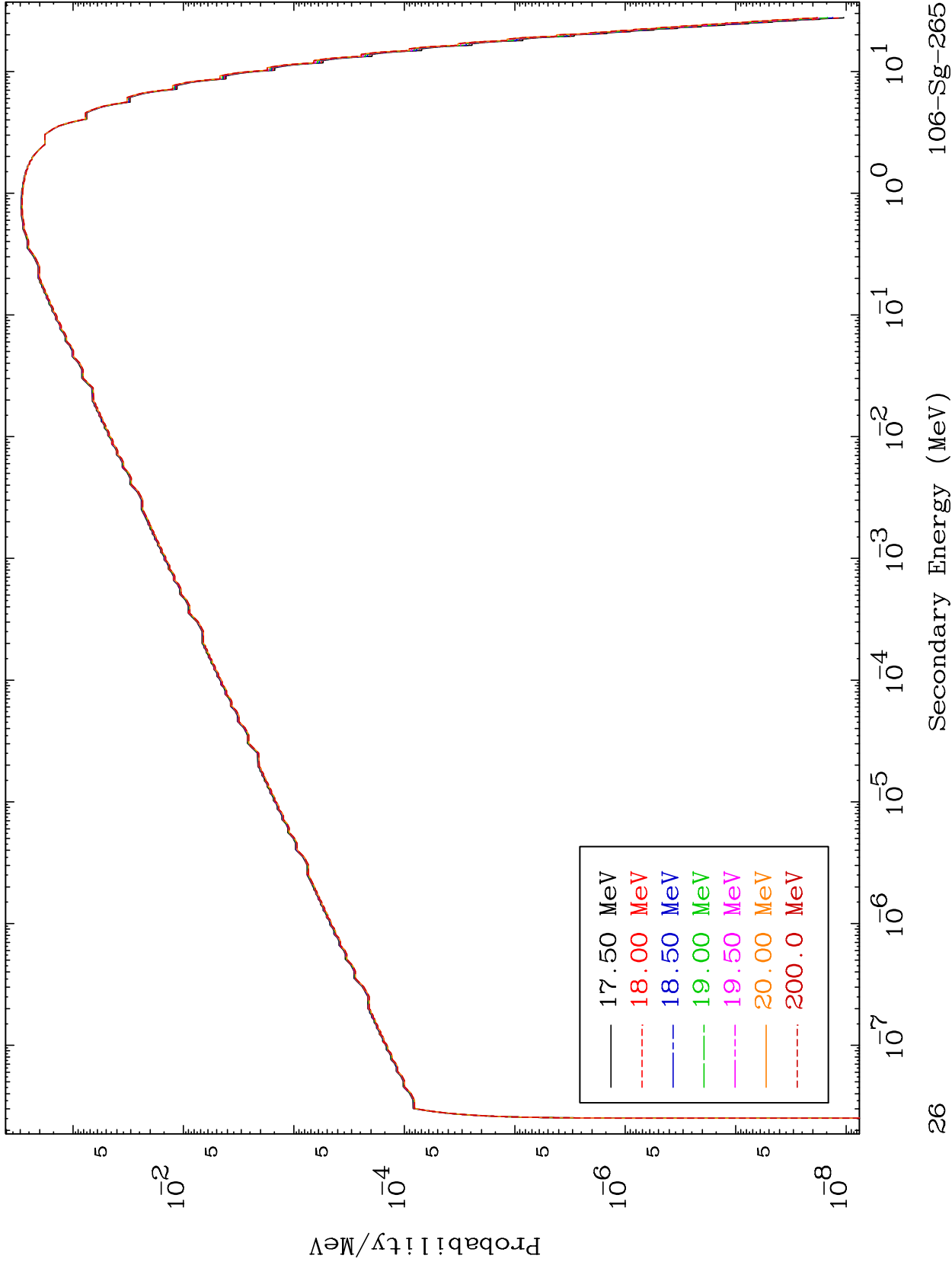
106-Sg-265

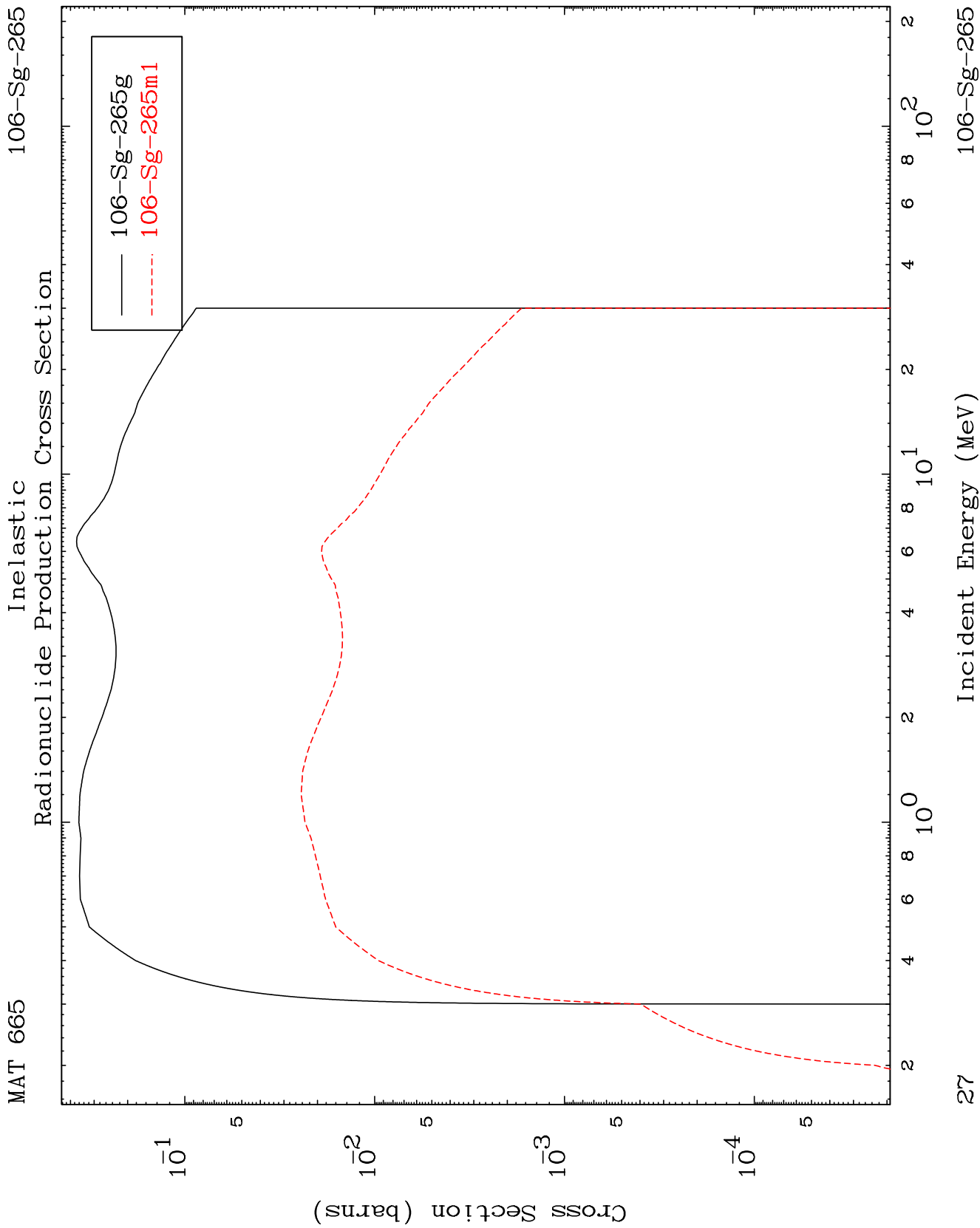


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### Fission Energy Distribution

106-Sg-265



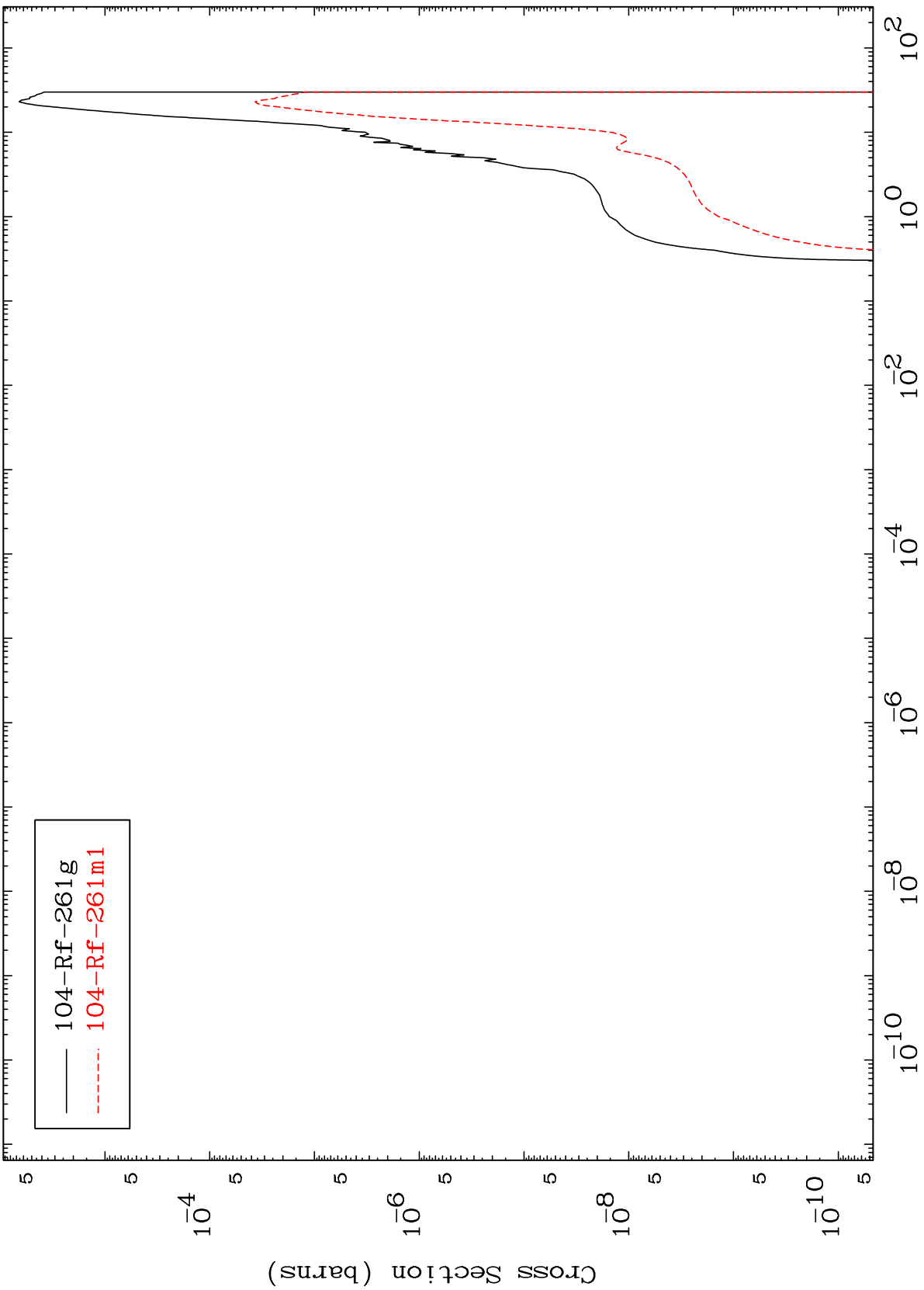


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$(n, n') \alpha$

106-Sg-265

Radionuclide Production Cross Section



MAT 665

(n,n') t,2 $\alpha$

106-Sg-265

Radionuclide Production Cross Section

