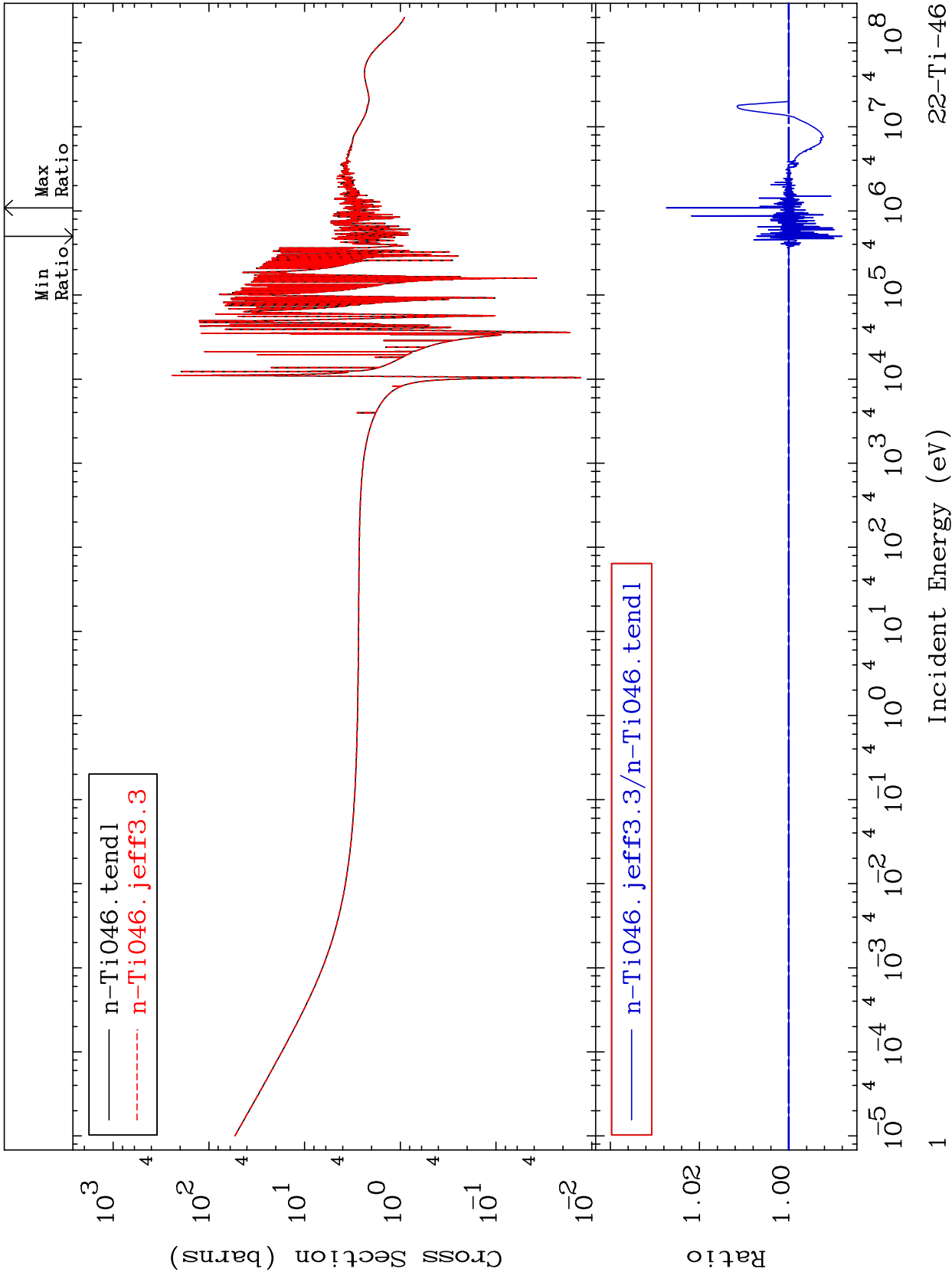


MAT 2225

Total
Cross Section

22-Ti-46
-1.194 To 2.738 %



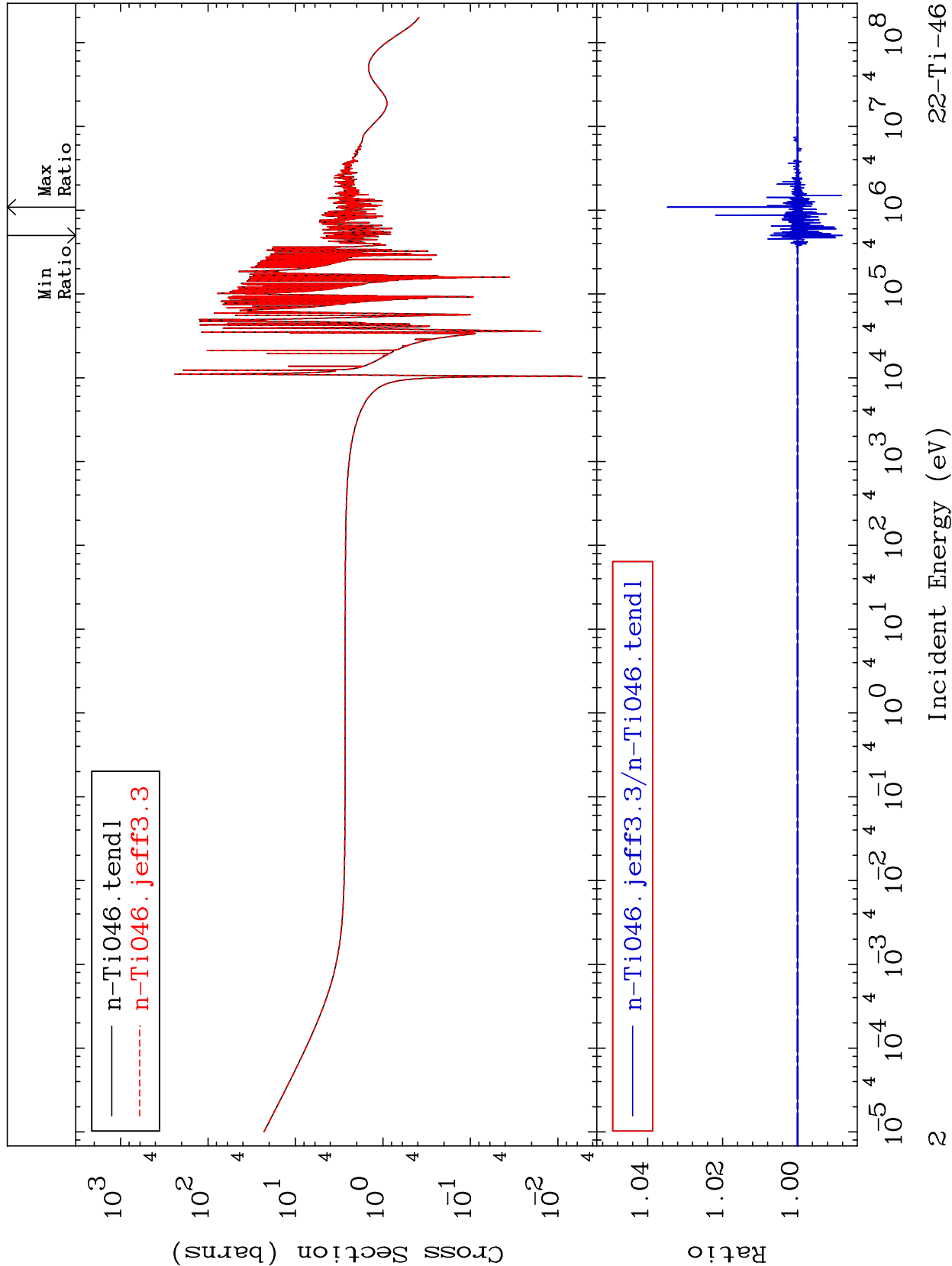
MAT 2225

Elastic

Cross Section

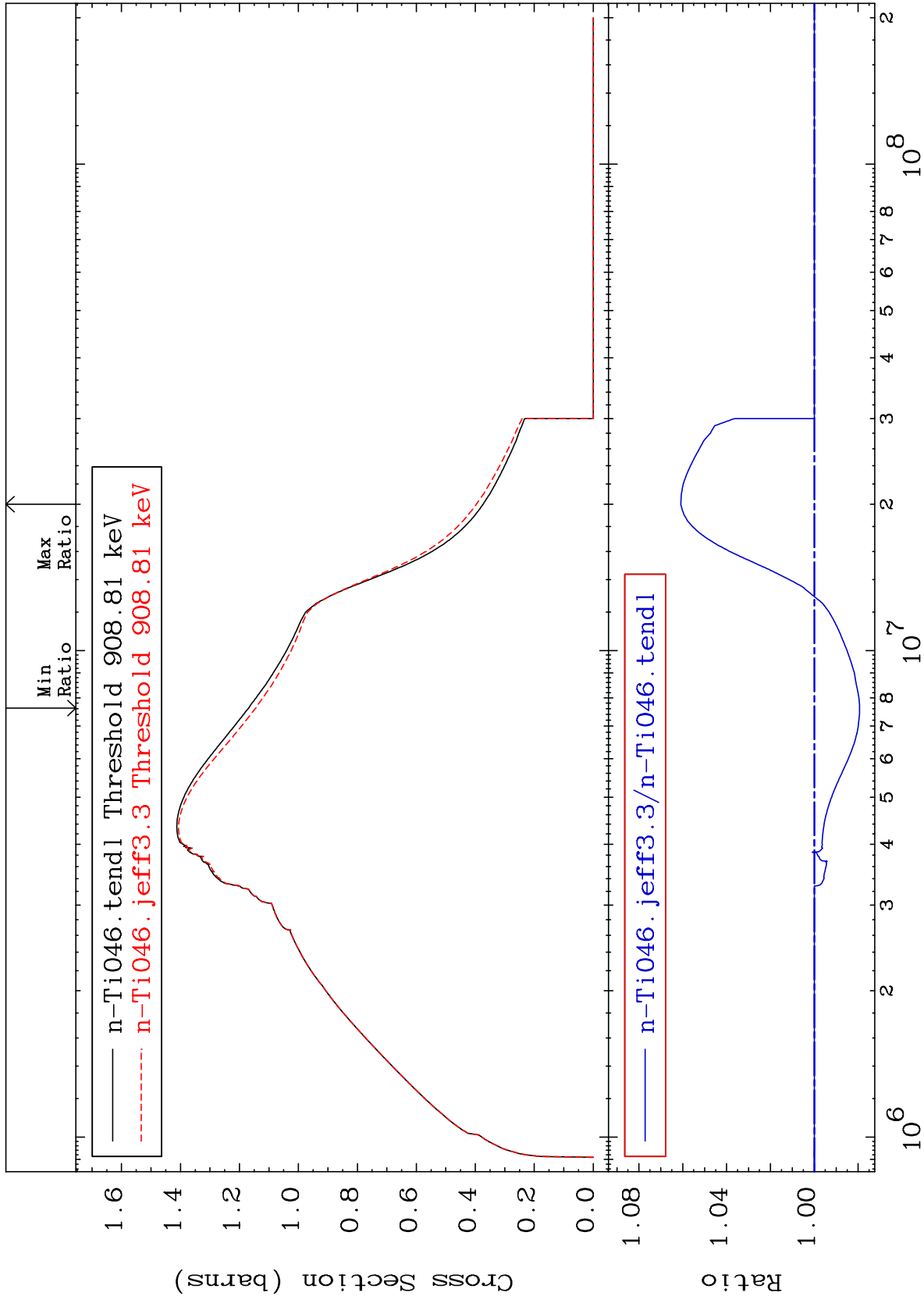
22-Ti-46

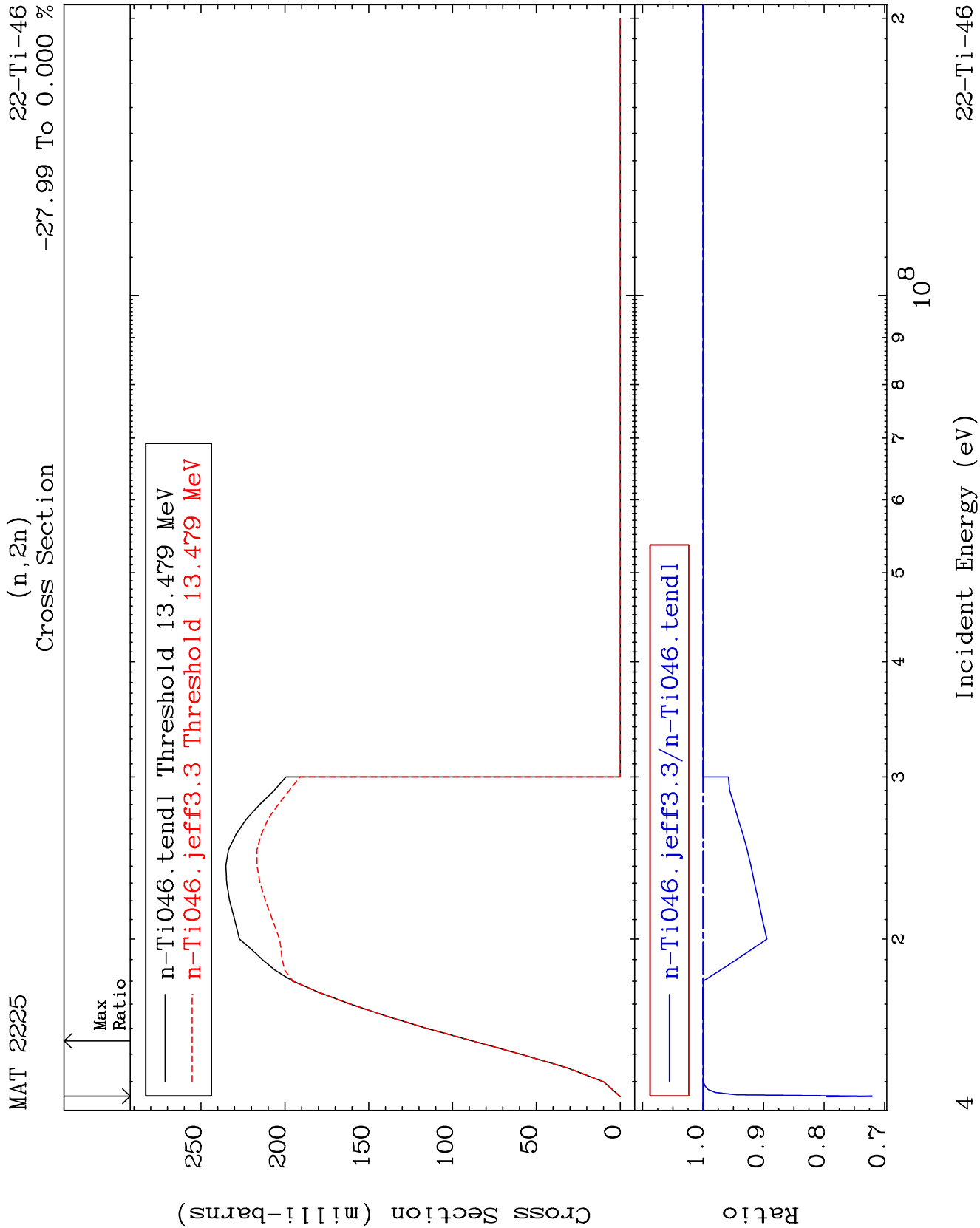
-1.196 To 3.470 %

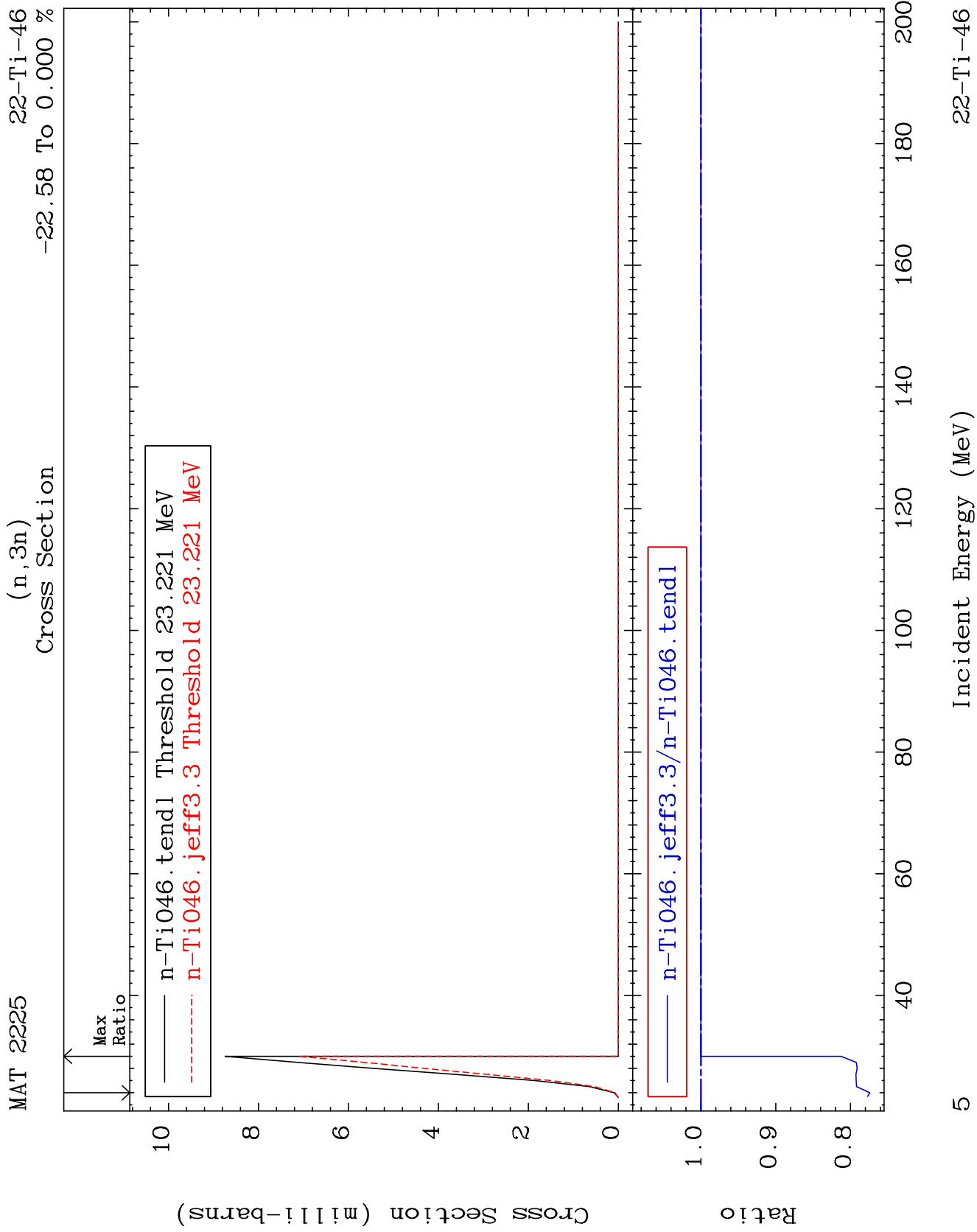


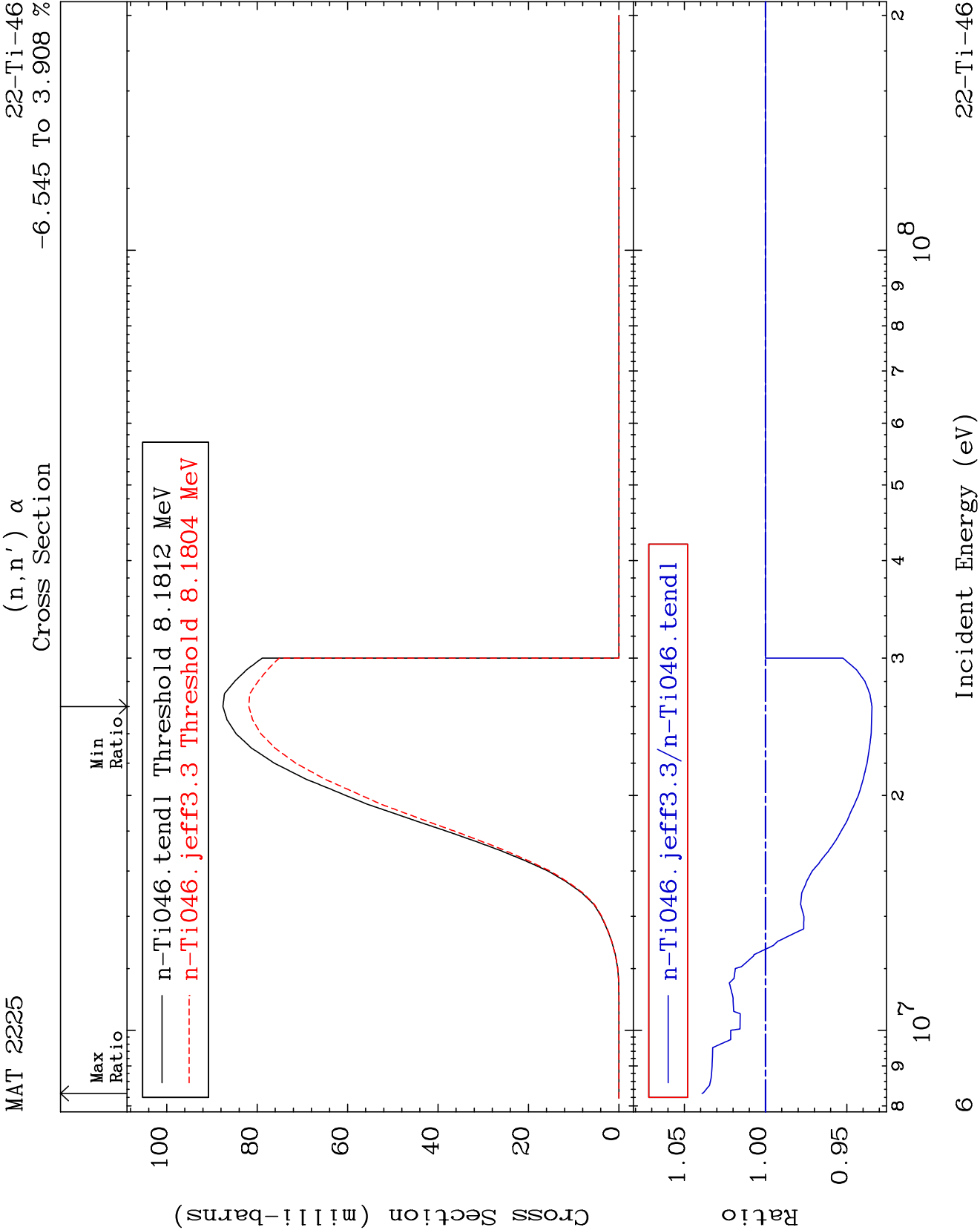
-2.063 To 6.090 %

Inelastic
Cross Section





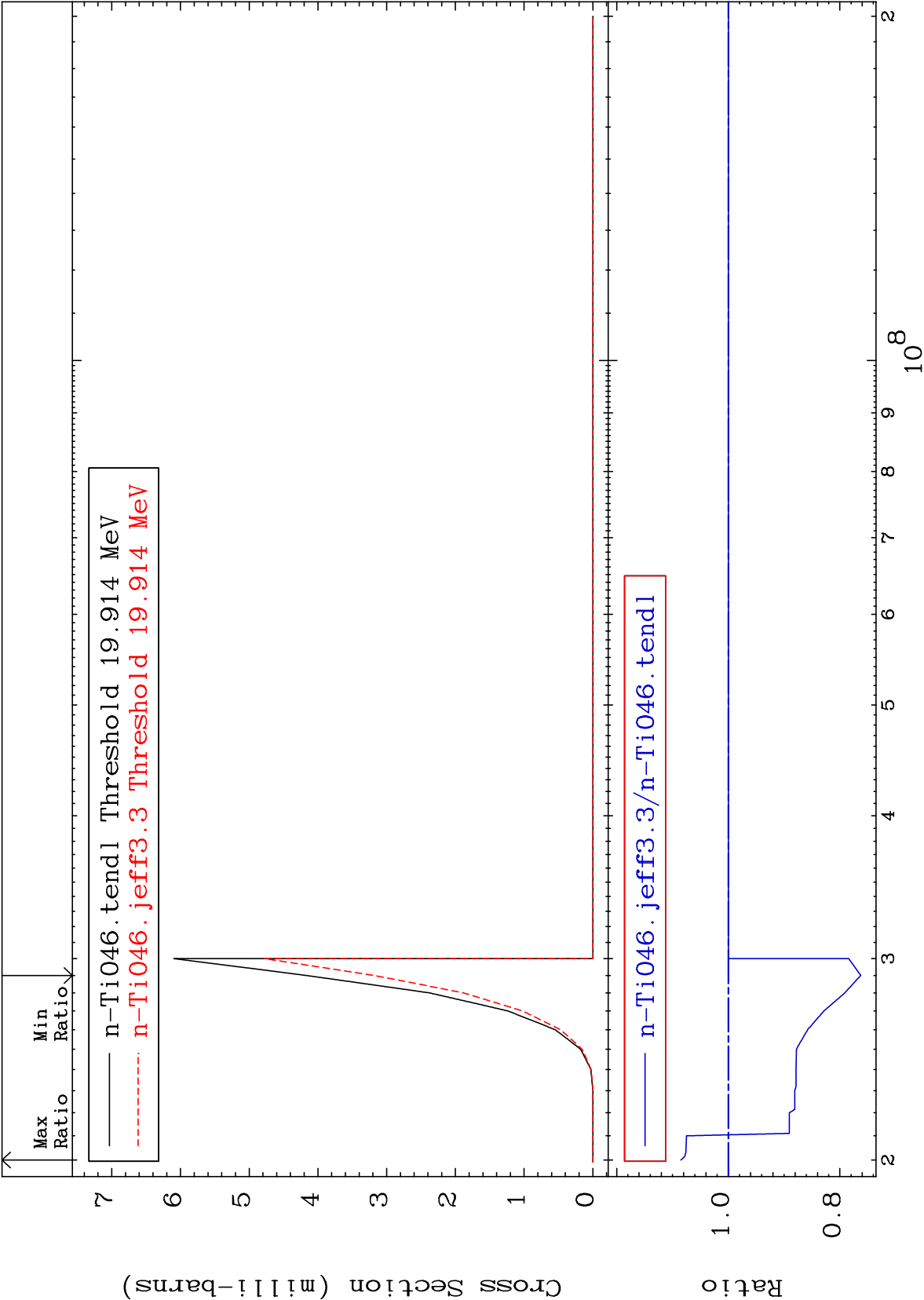


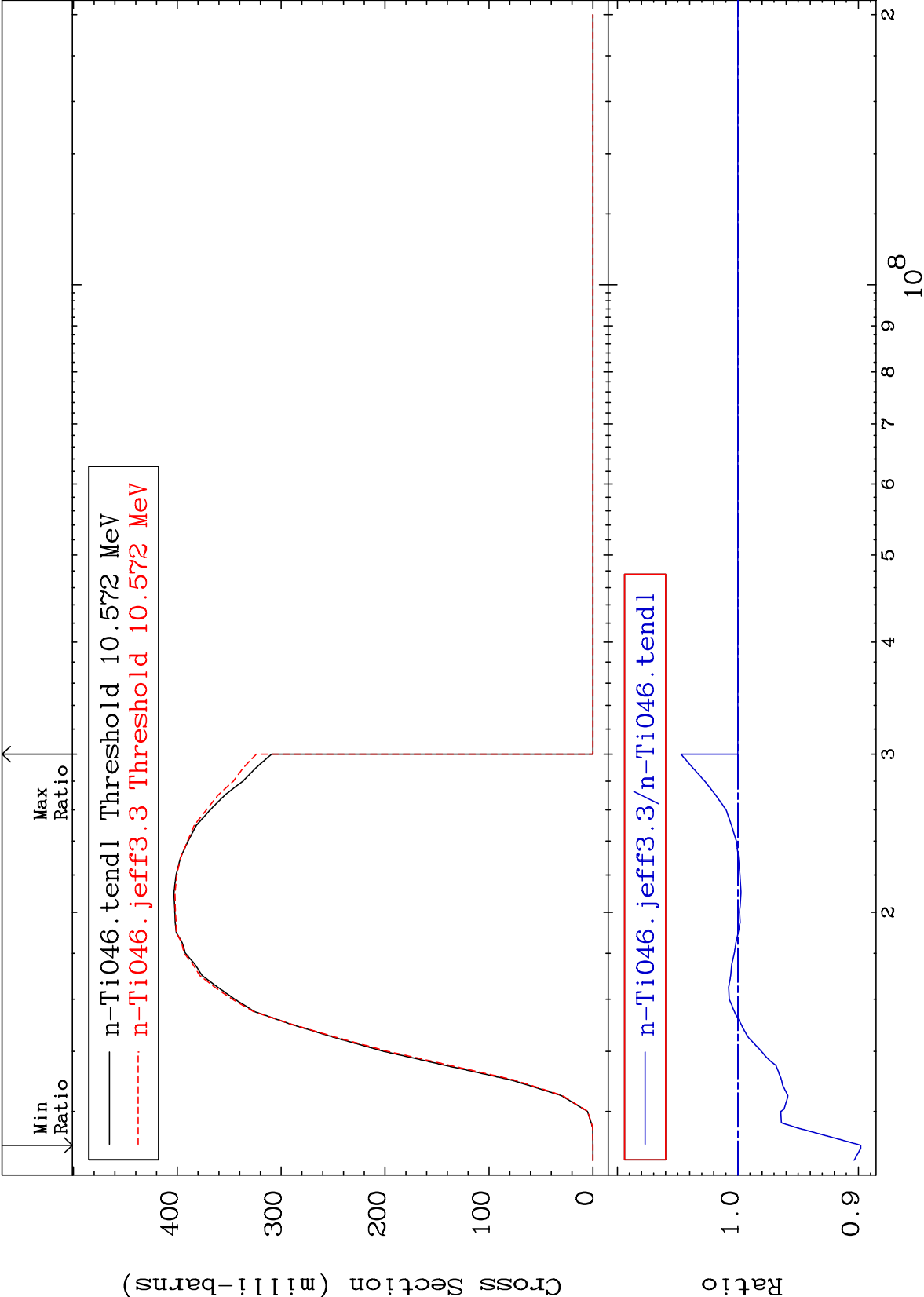


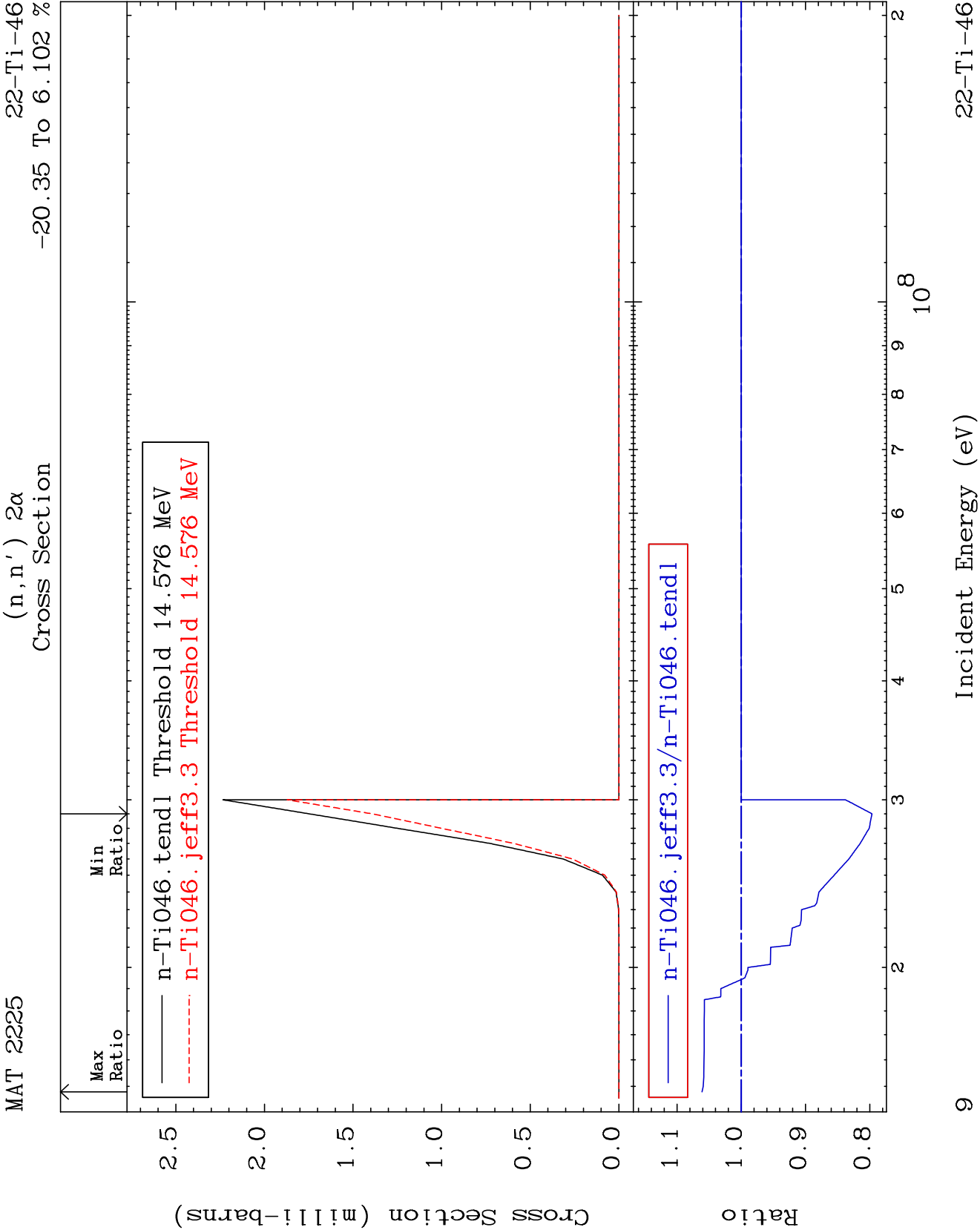
MAT 2225

(n,2n) α
Cross Section

22-Ti-46
-23.74 To 8.496 %







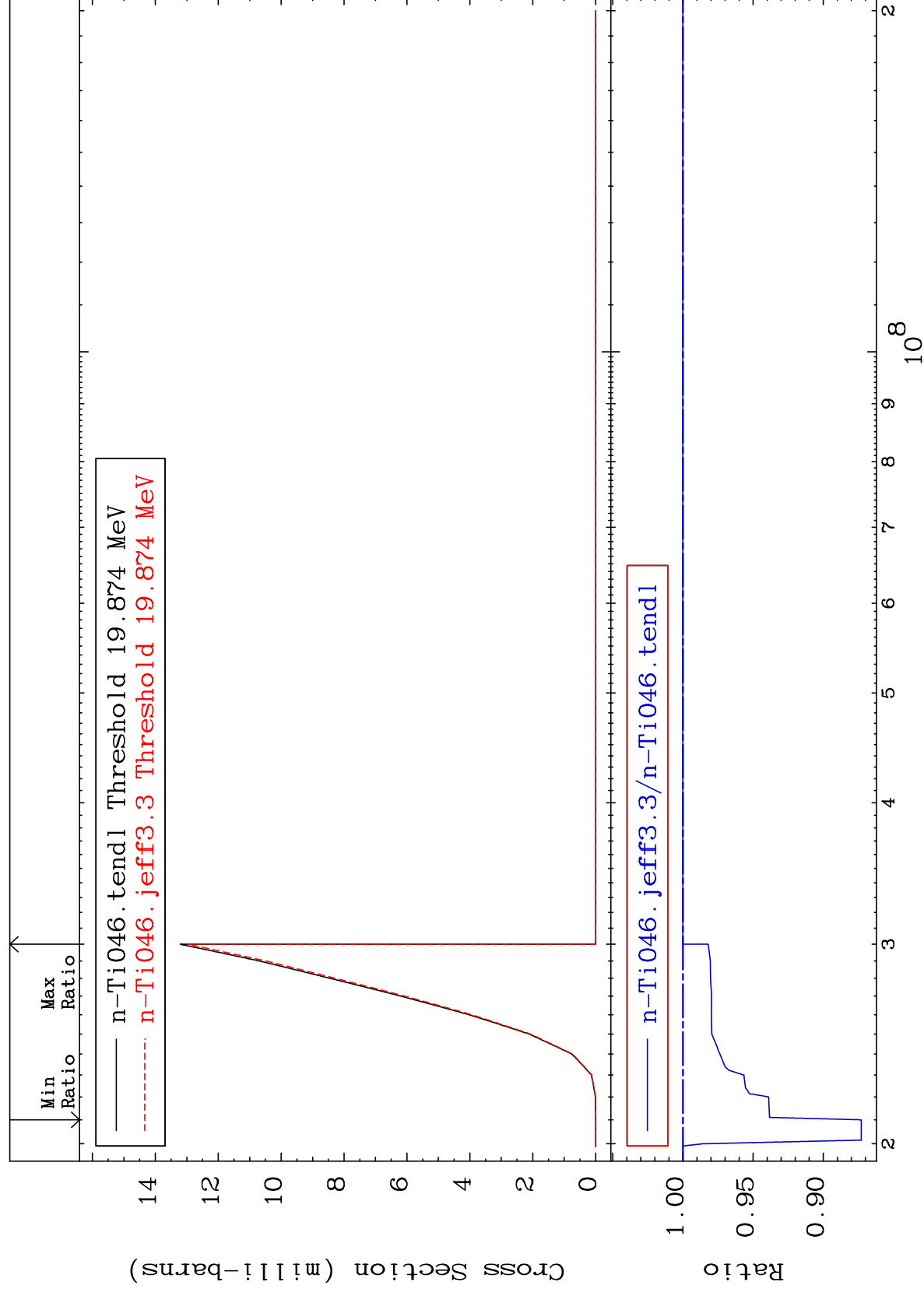
MAT 2225

$$(n, n')$$

Cross Section

22-Ti-46

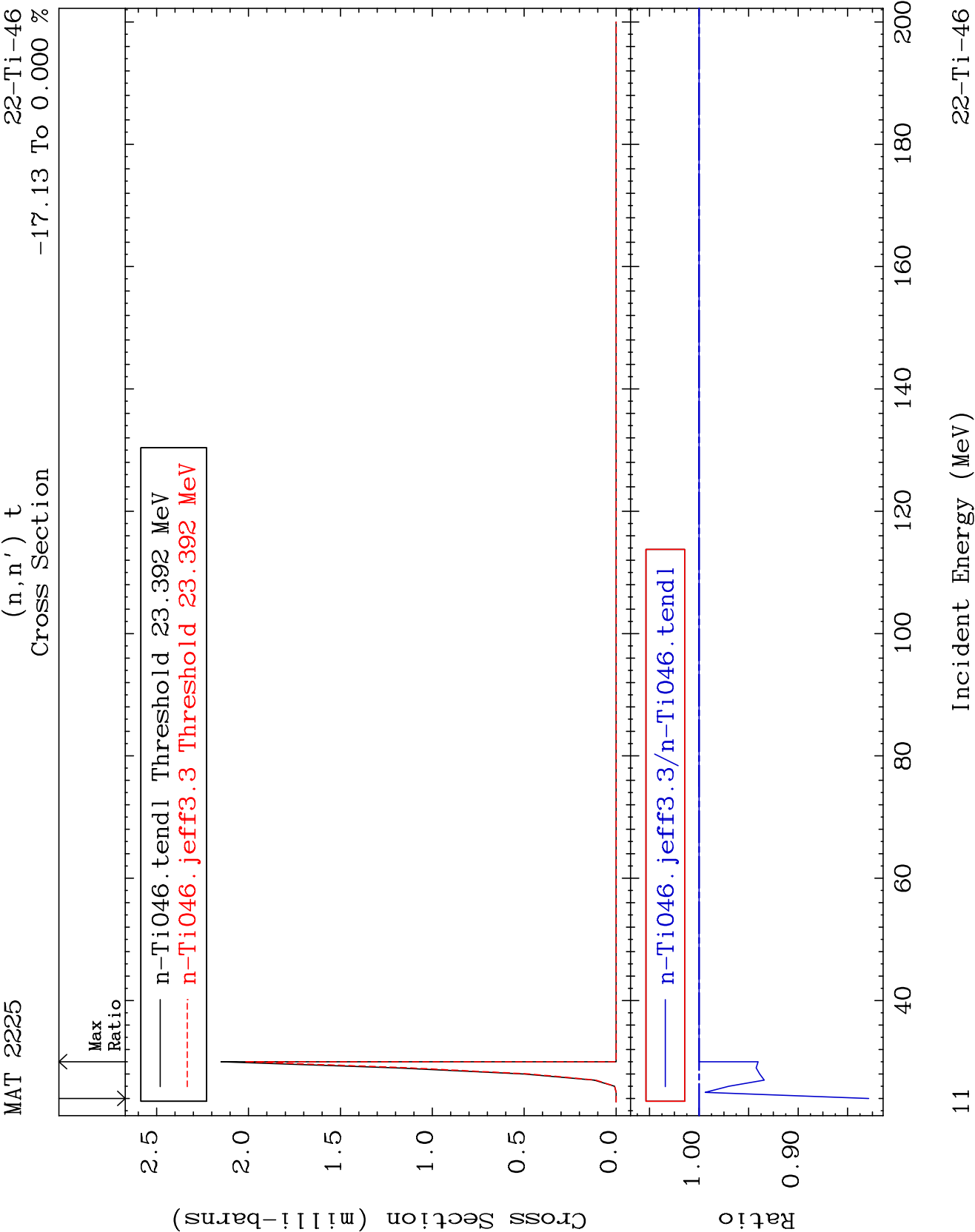
-12.71 To 0.000 %



10

Incident Energy (eV)

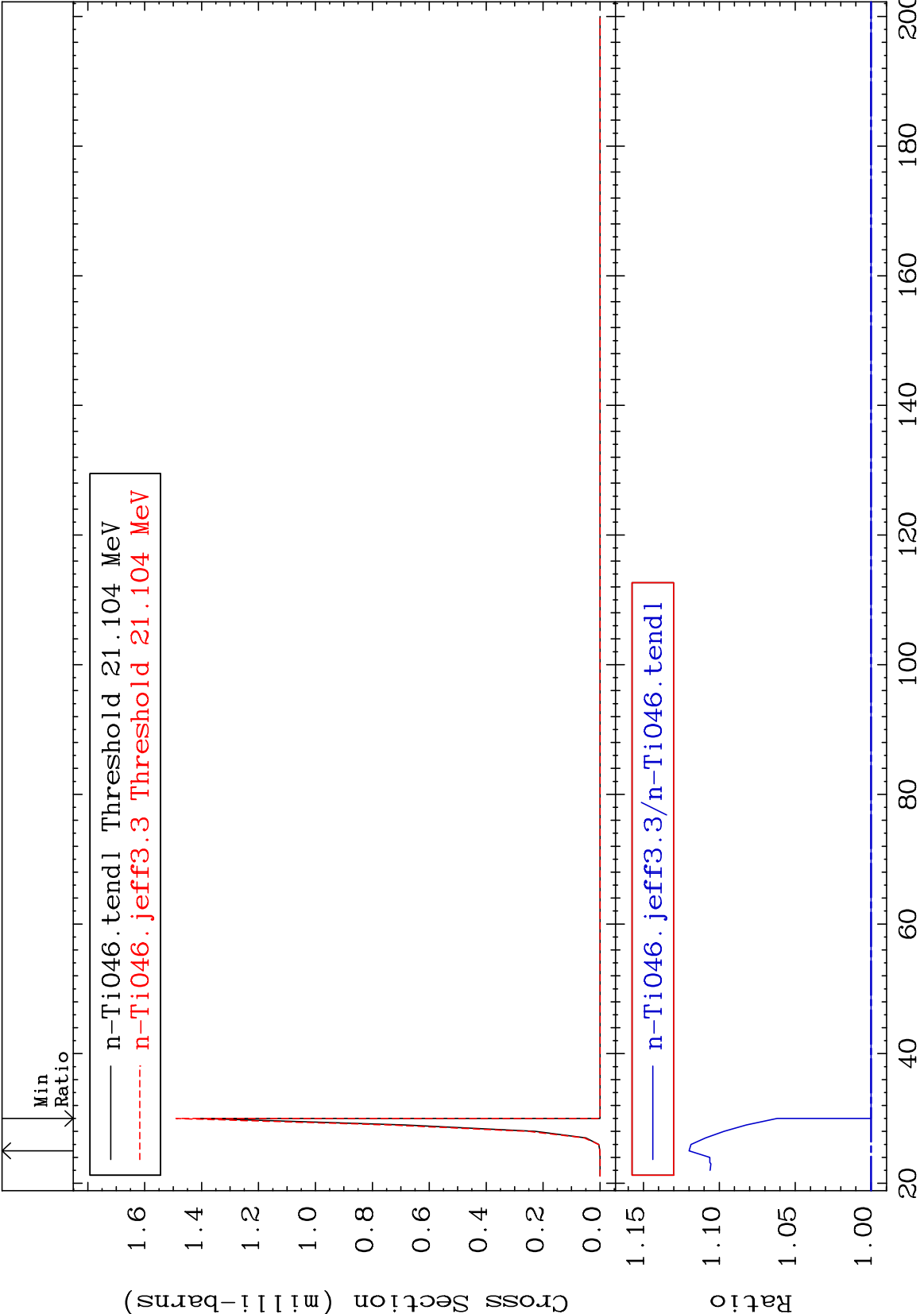
22-Ti-46



MAT 2225

(n,n') He-3
Cross Section

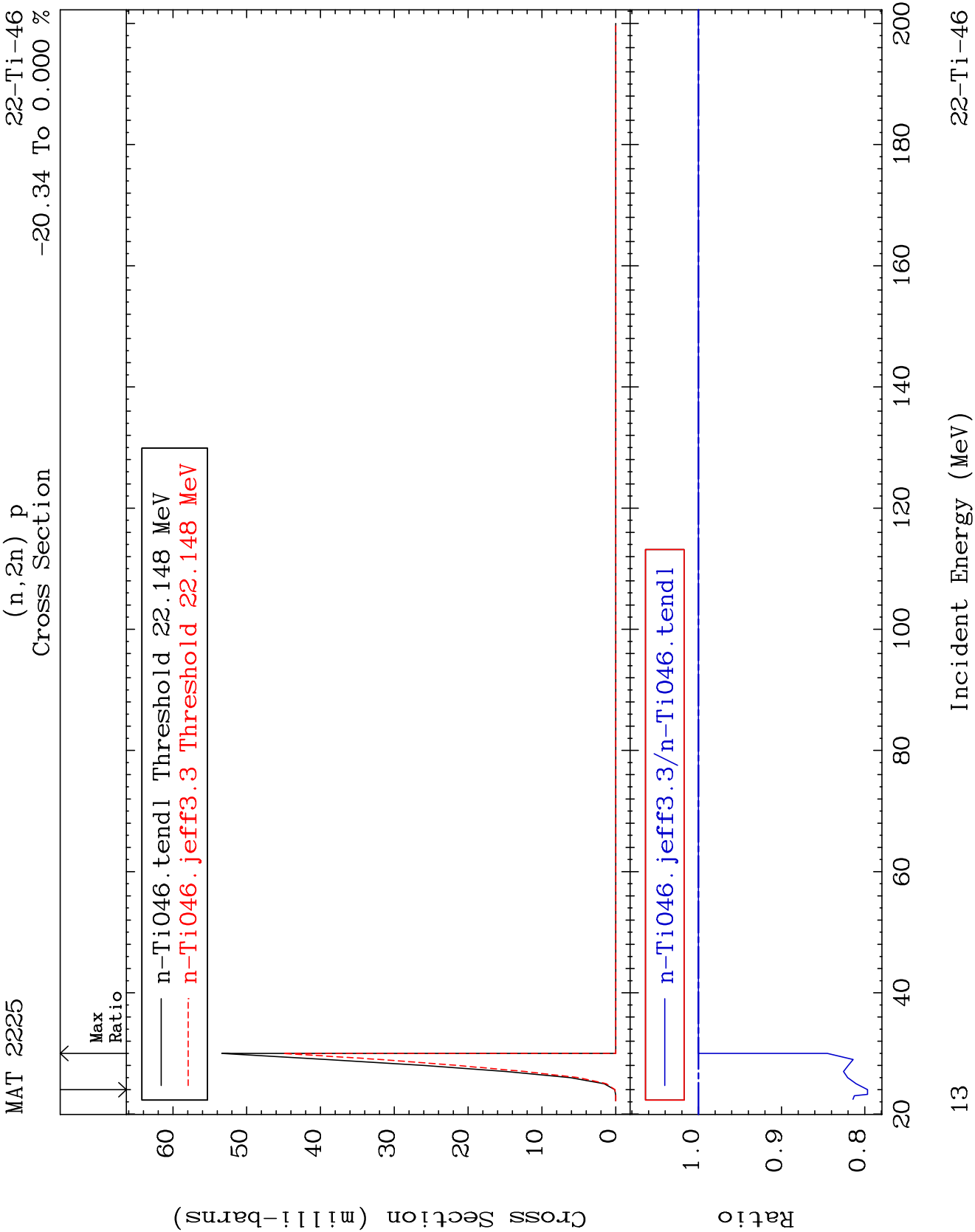
22-Ti-46
0.000 To 11.99 %



12

Incident Energy (MeV)

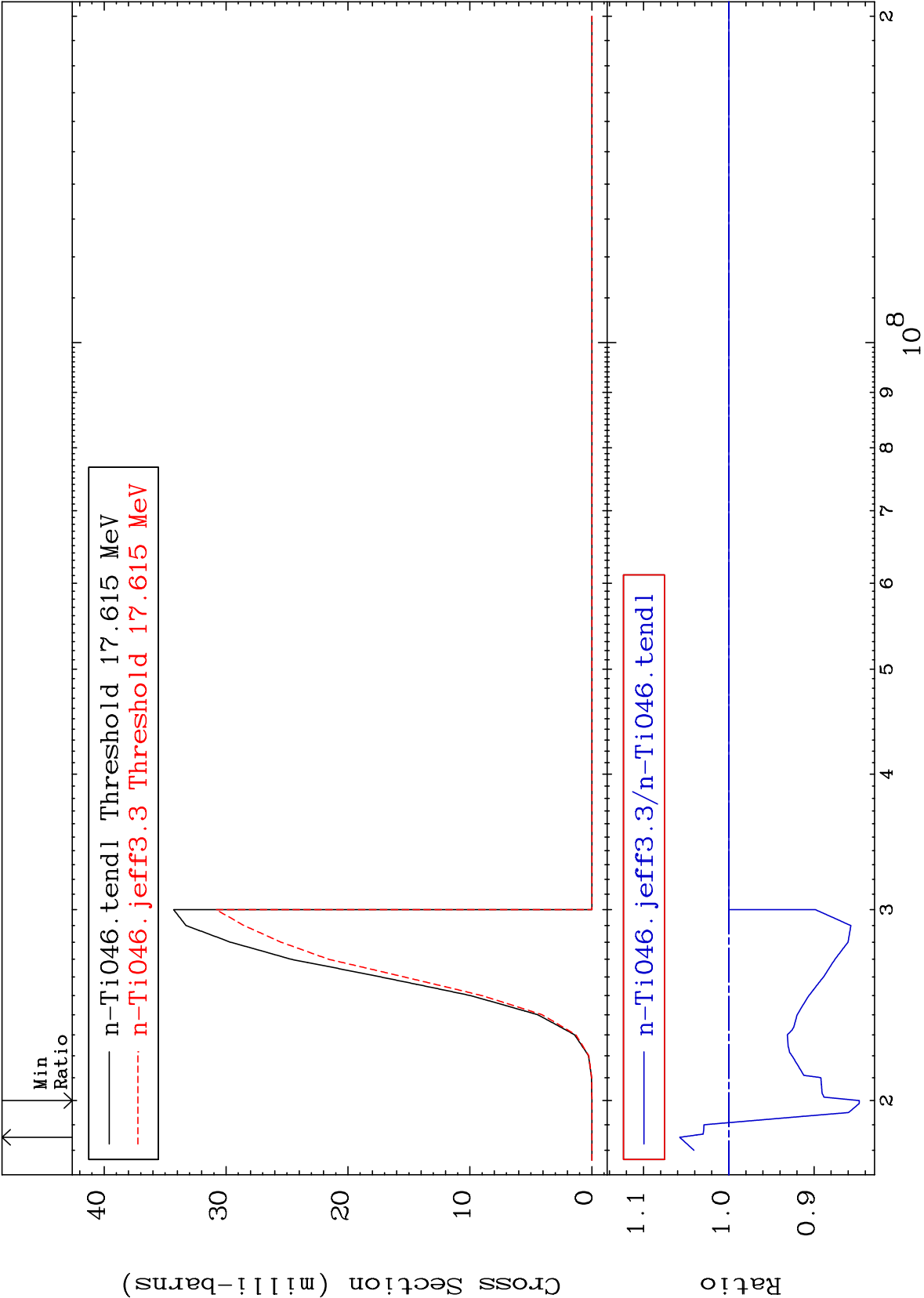
22-Ti-46

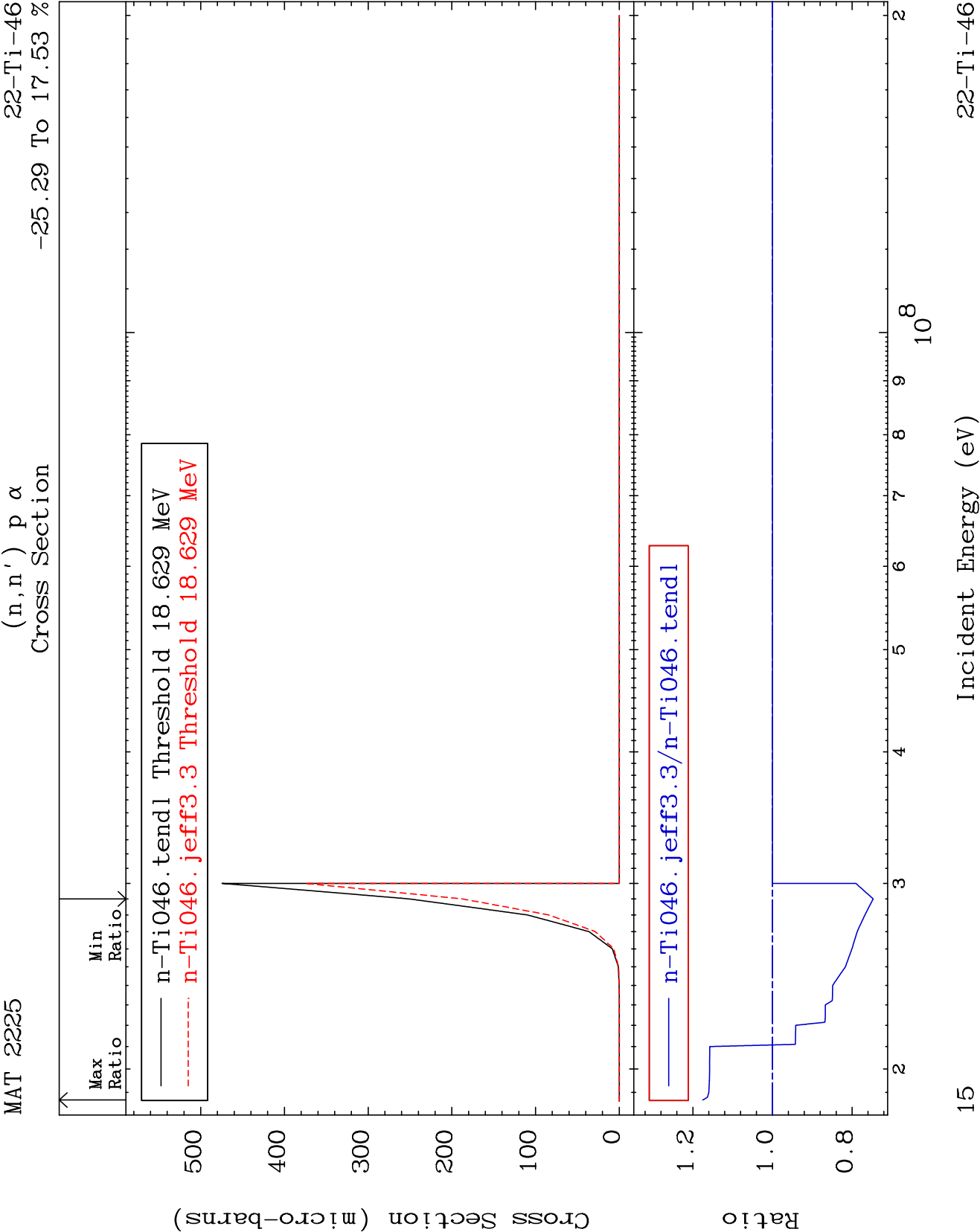


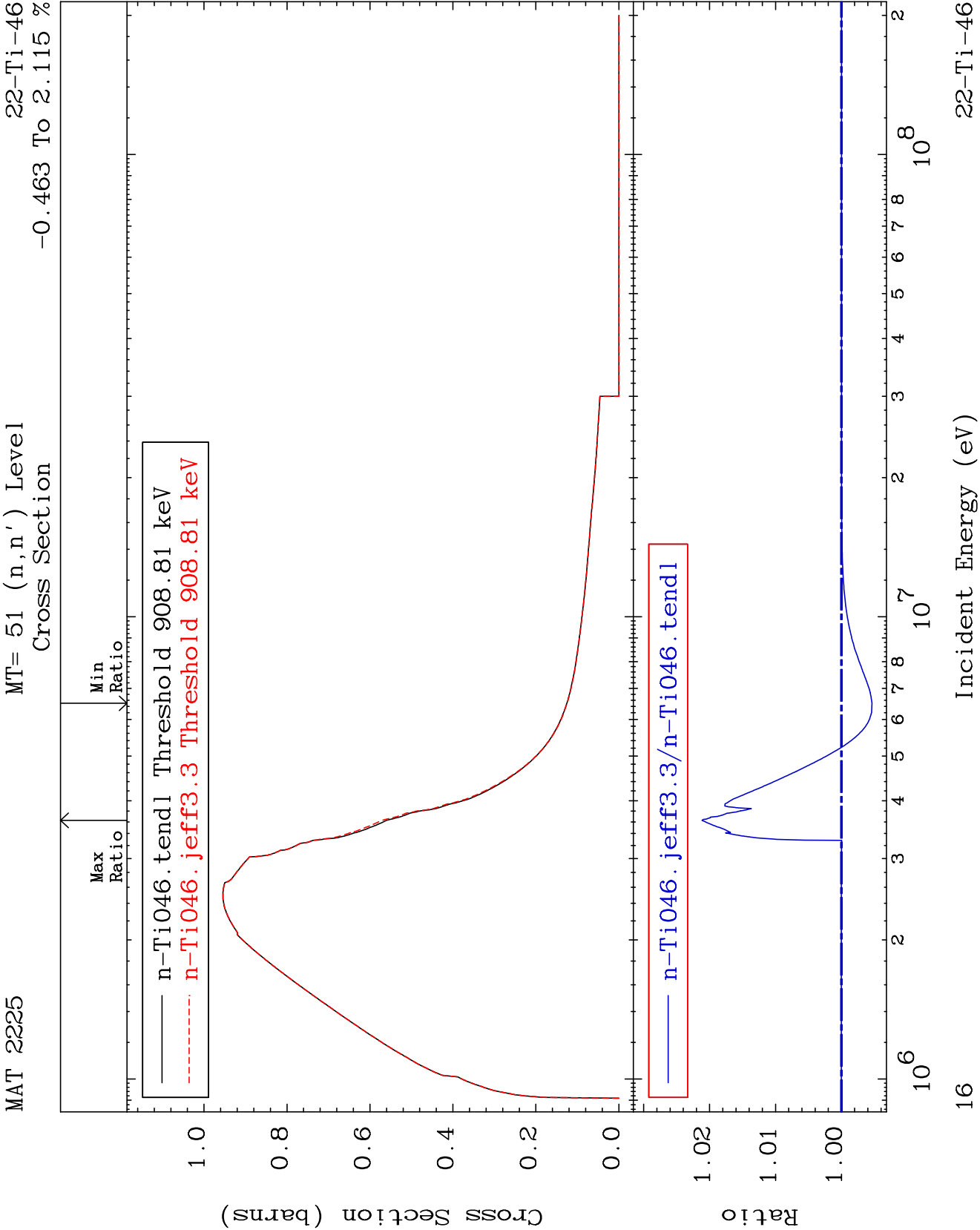
MAT 2225

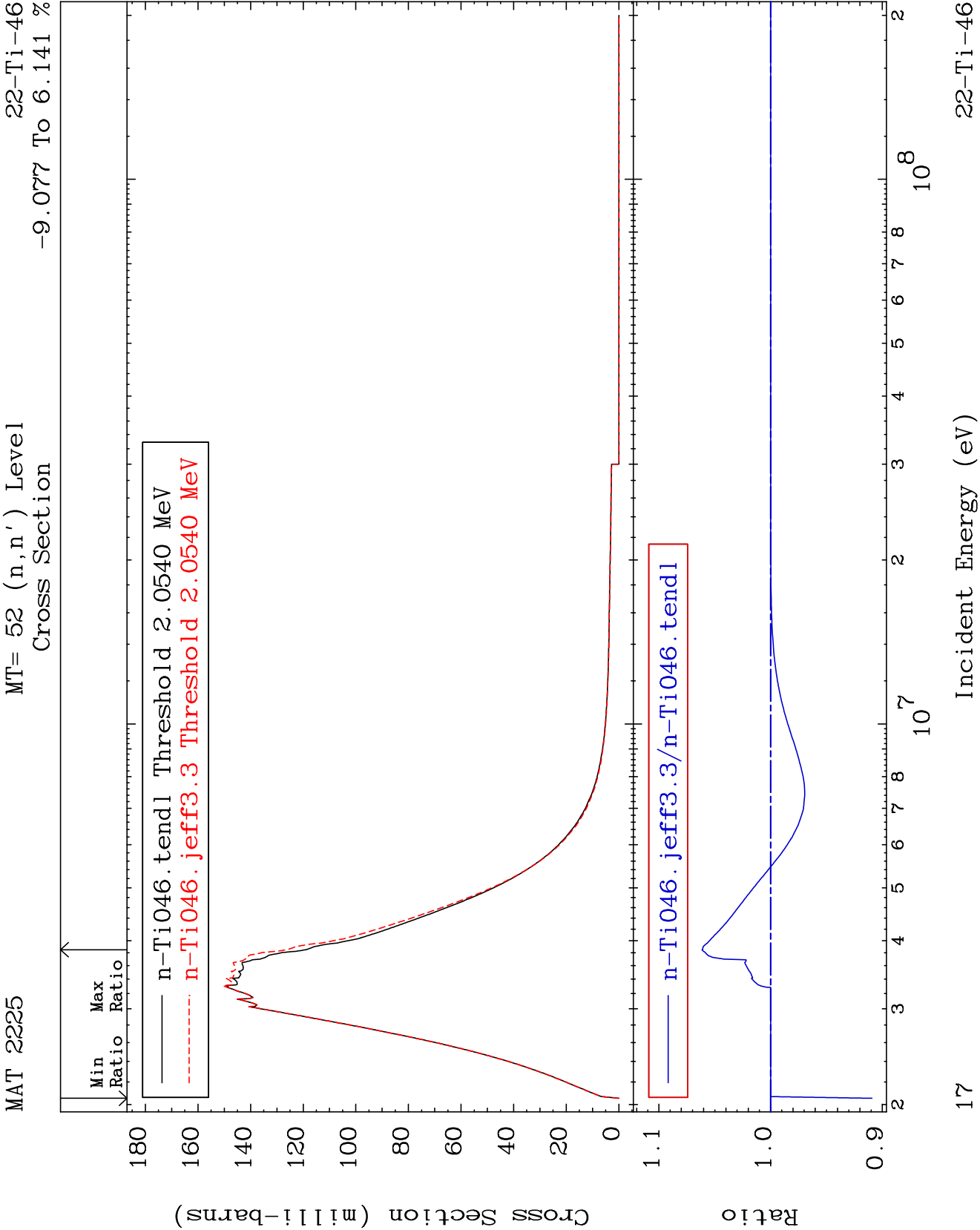
(n,2n) p
Cross Section

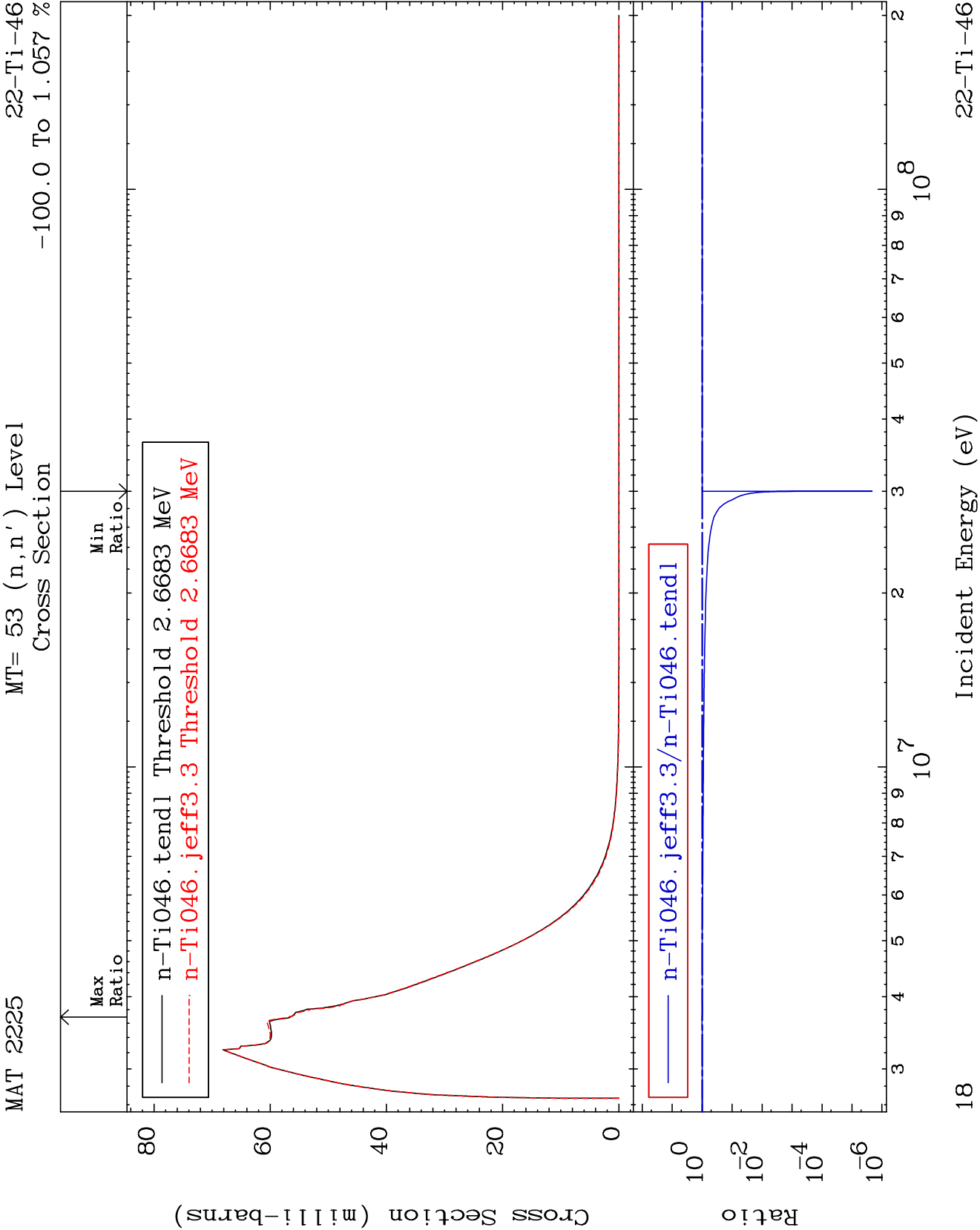
22-Ti-46
-15.30 To 5.753 %

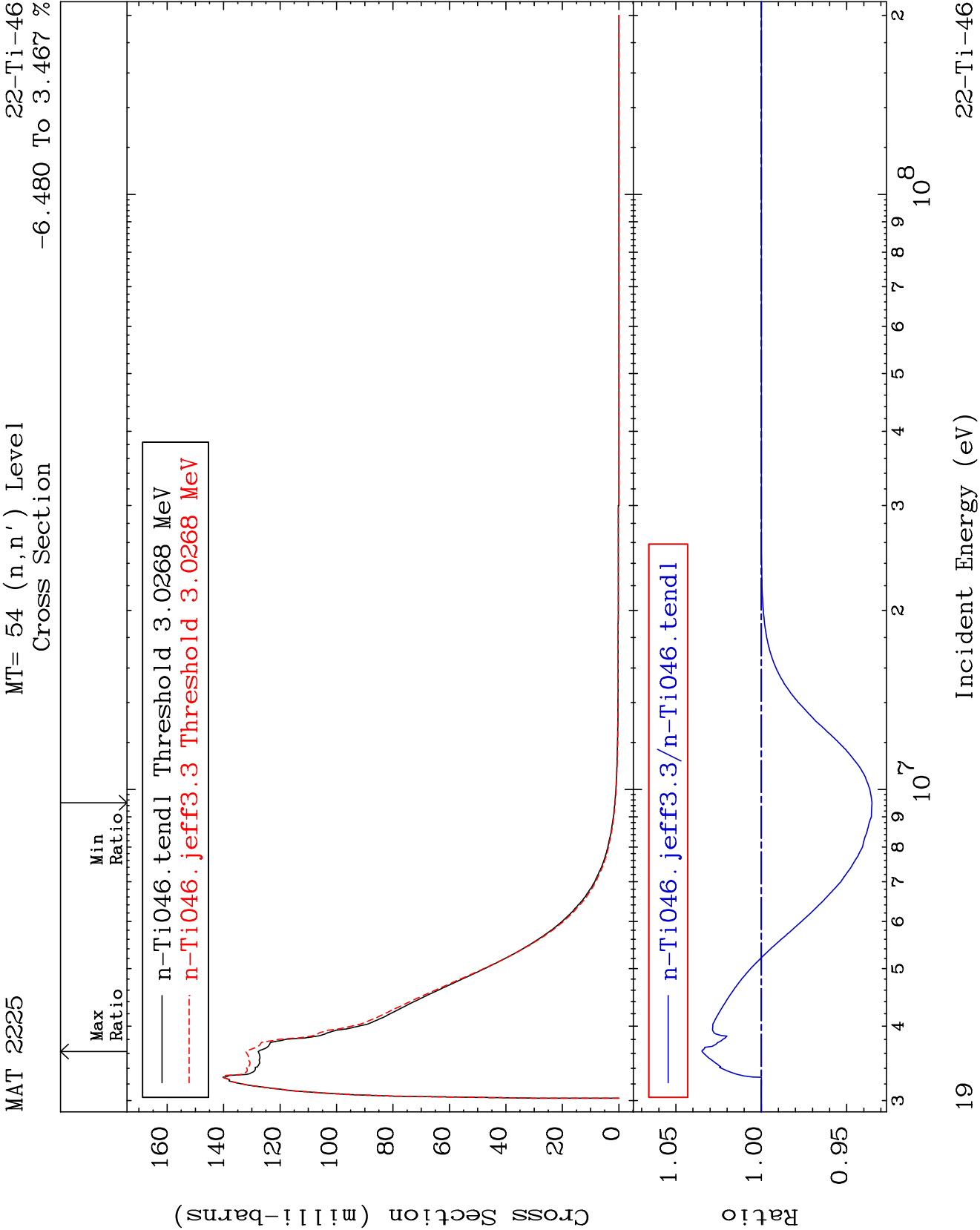








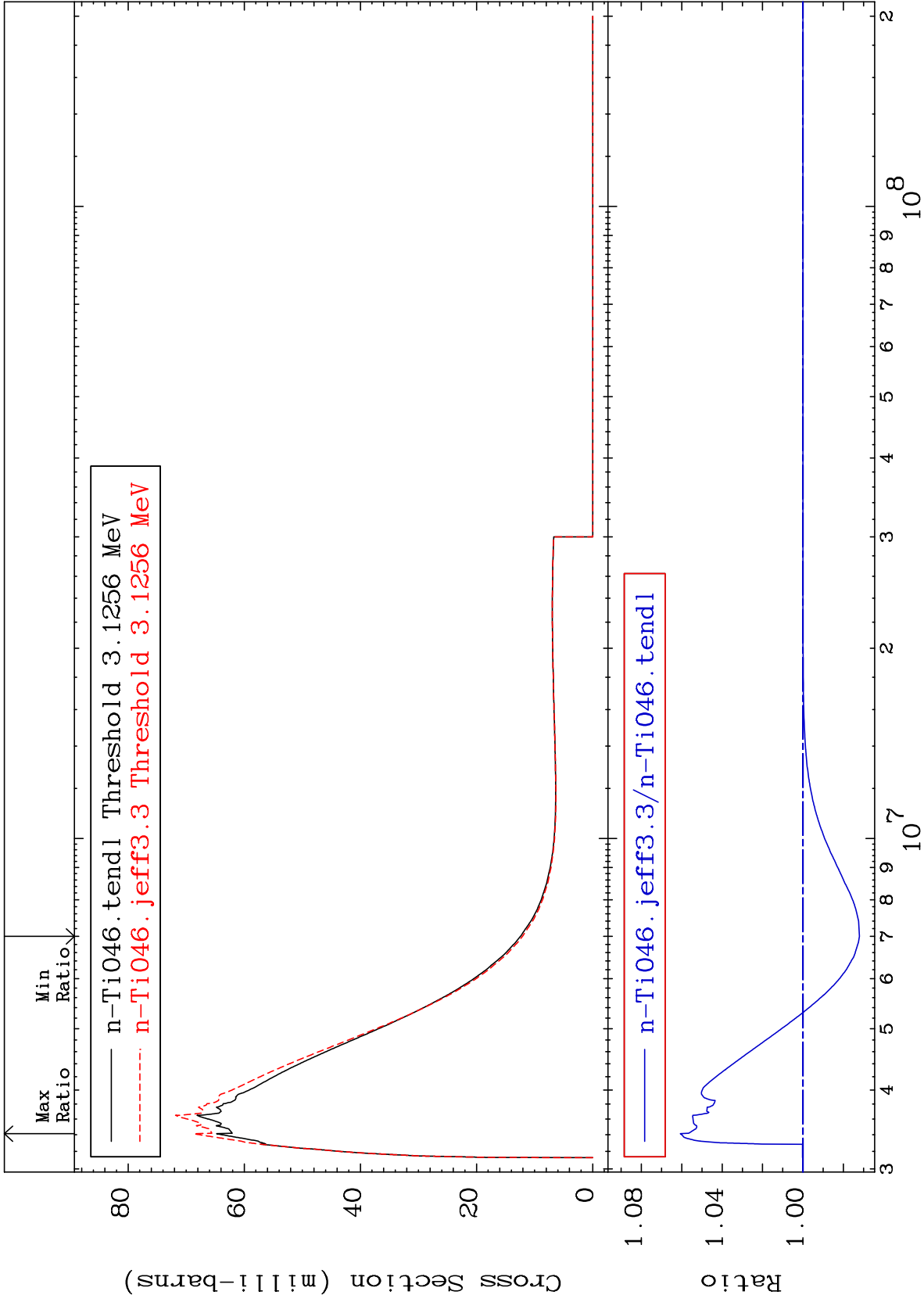




MAT 2225

MT= 55 (n,n') Level
Cross Section

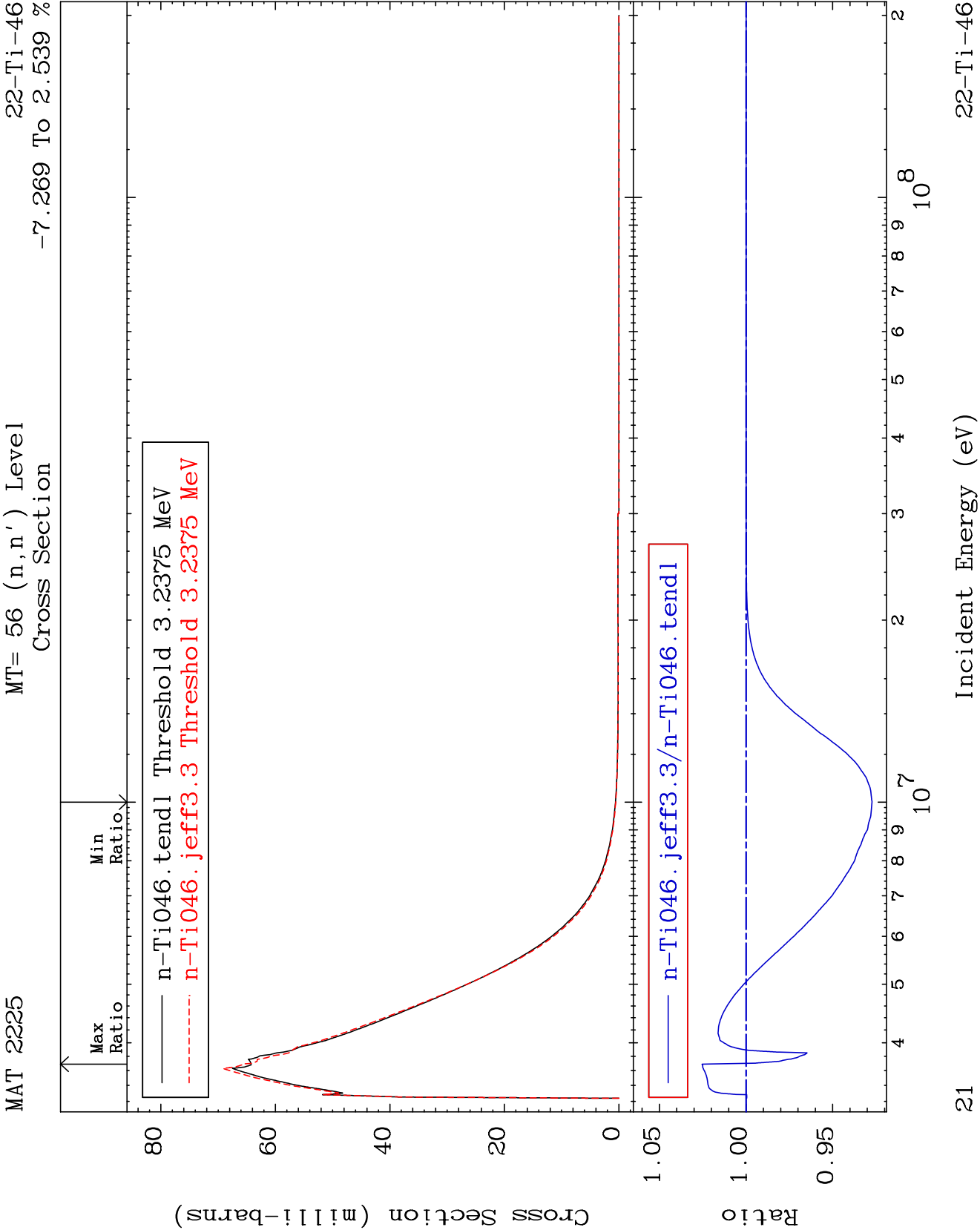
22-Ti-46
-2.799 To 6.070 %



20

Incident Energy (eV)

22-Ti-46



21

22-Ti-46

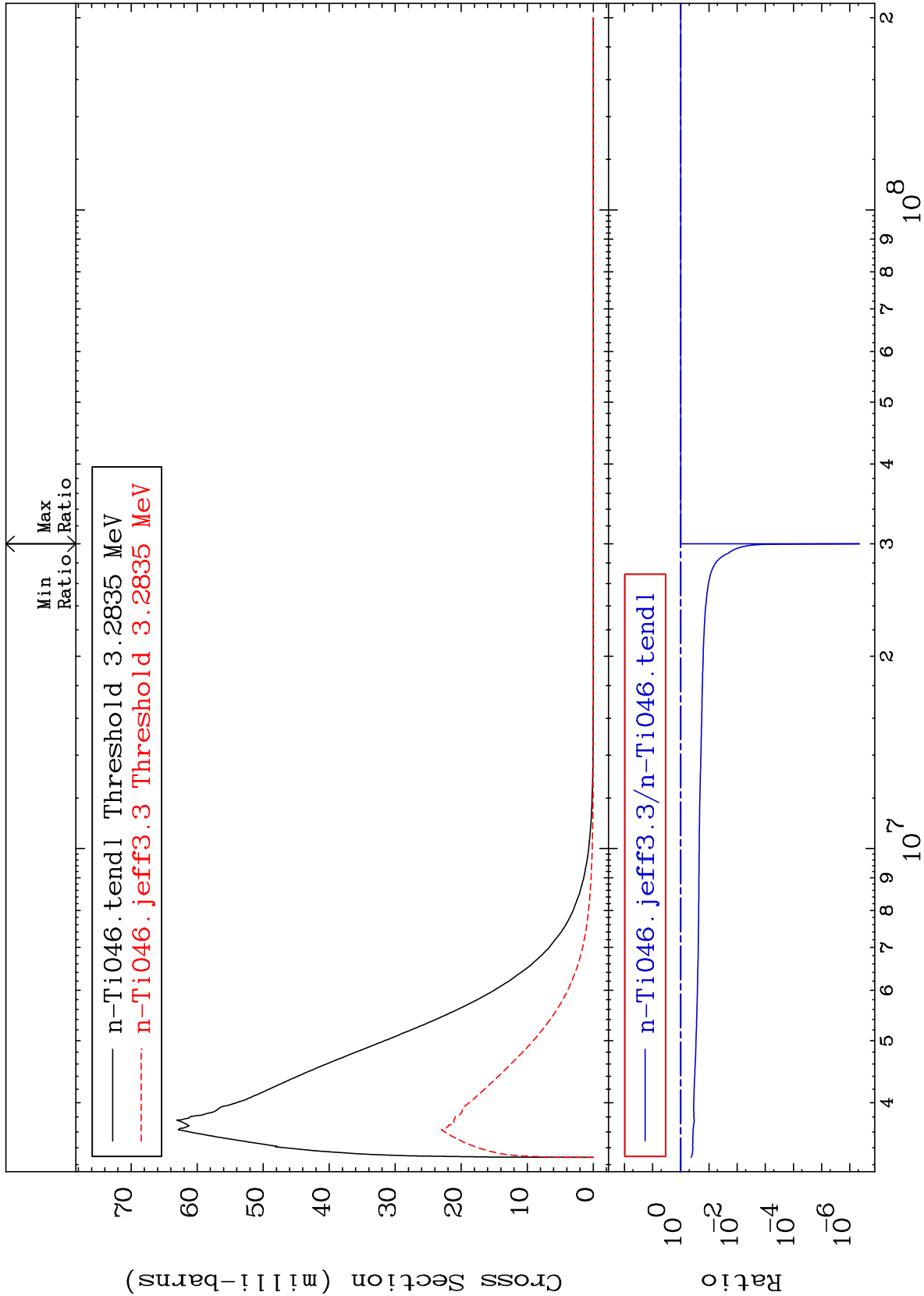
MAT 2225

MT= 57 (n,n') Level

22-Ti-46

-100.0 To 0.000 %

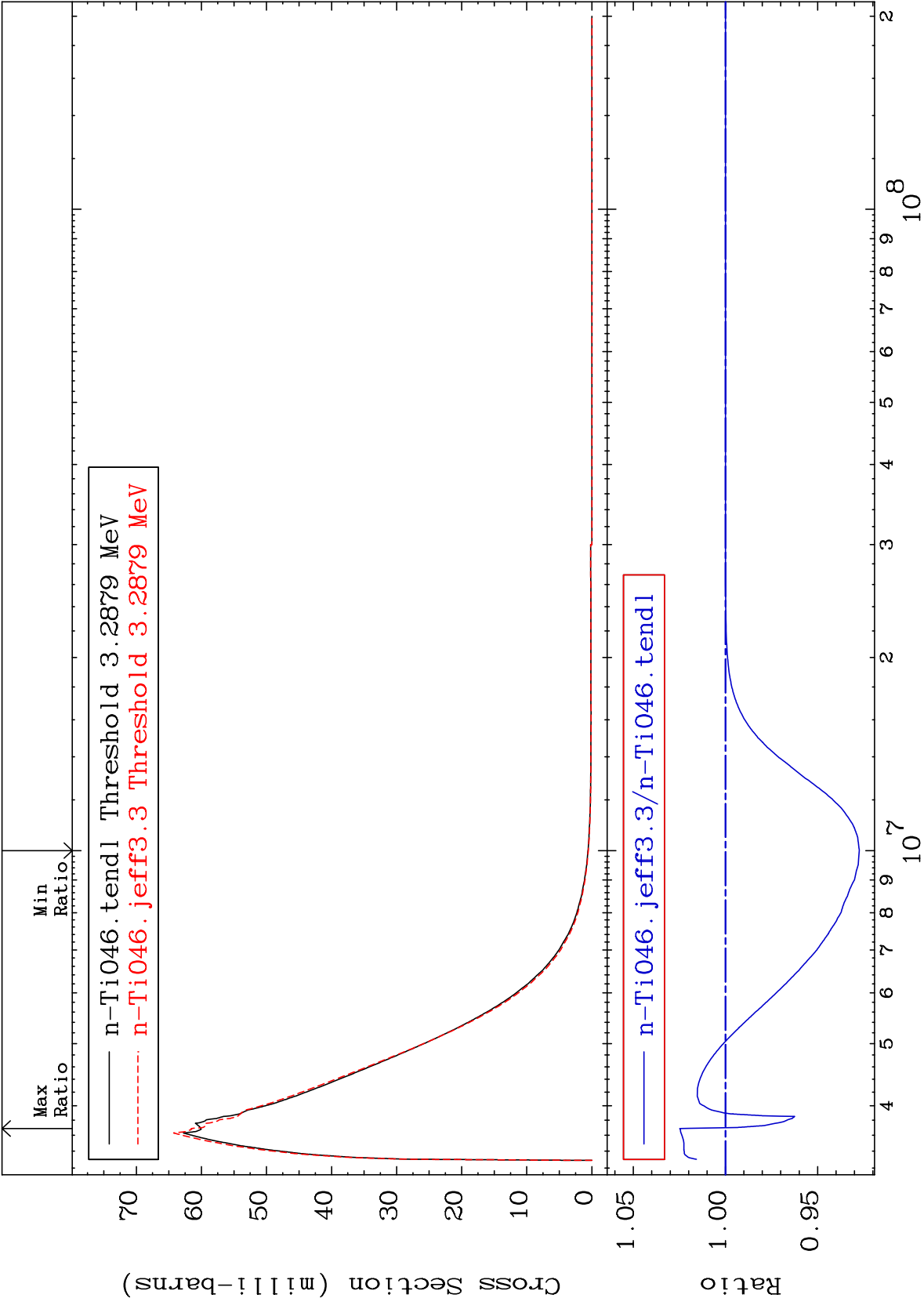
Cross Section

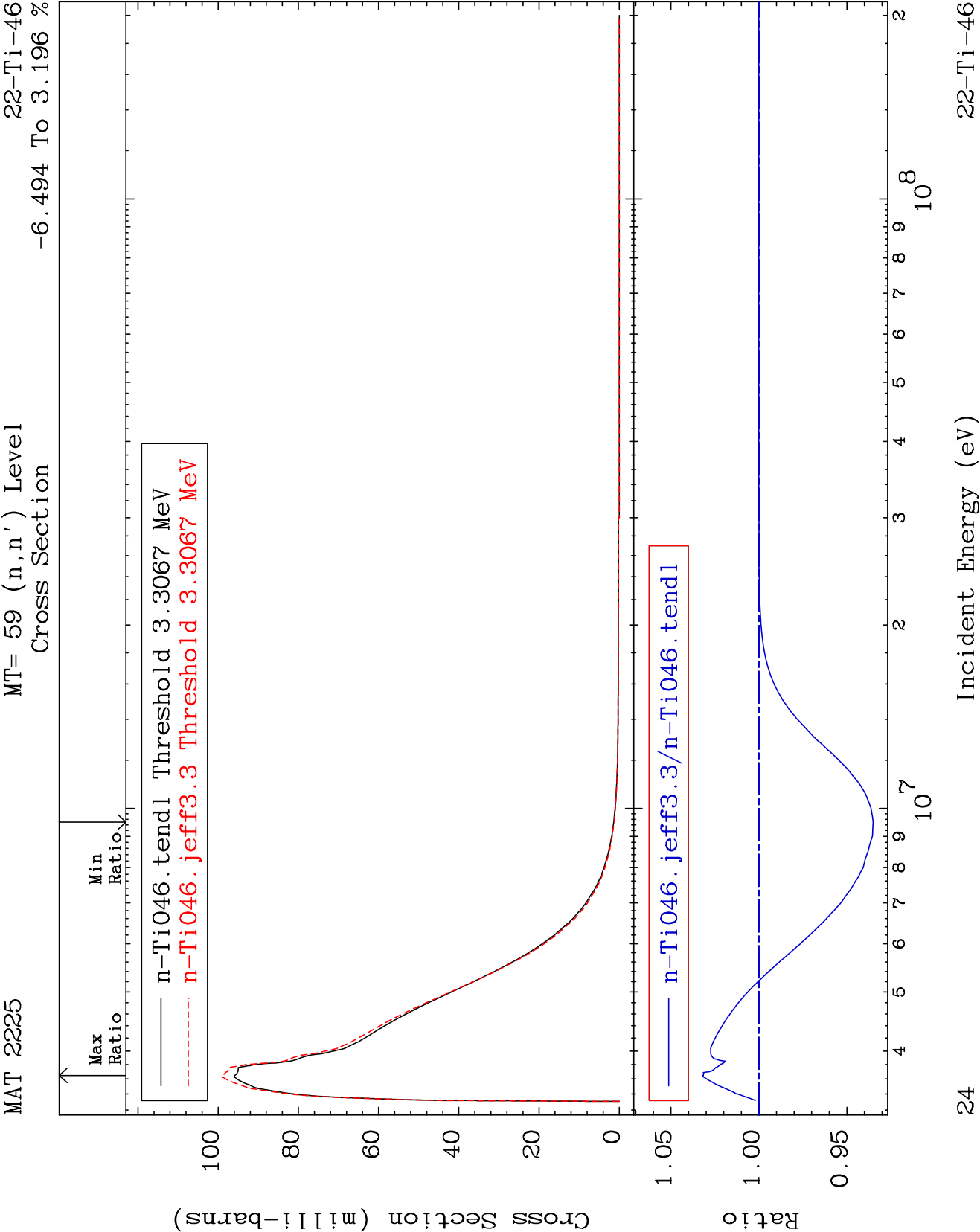


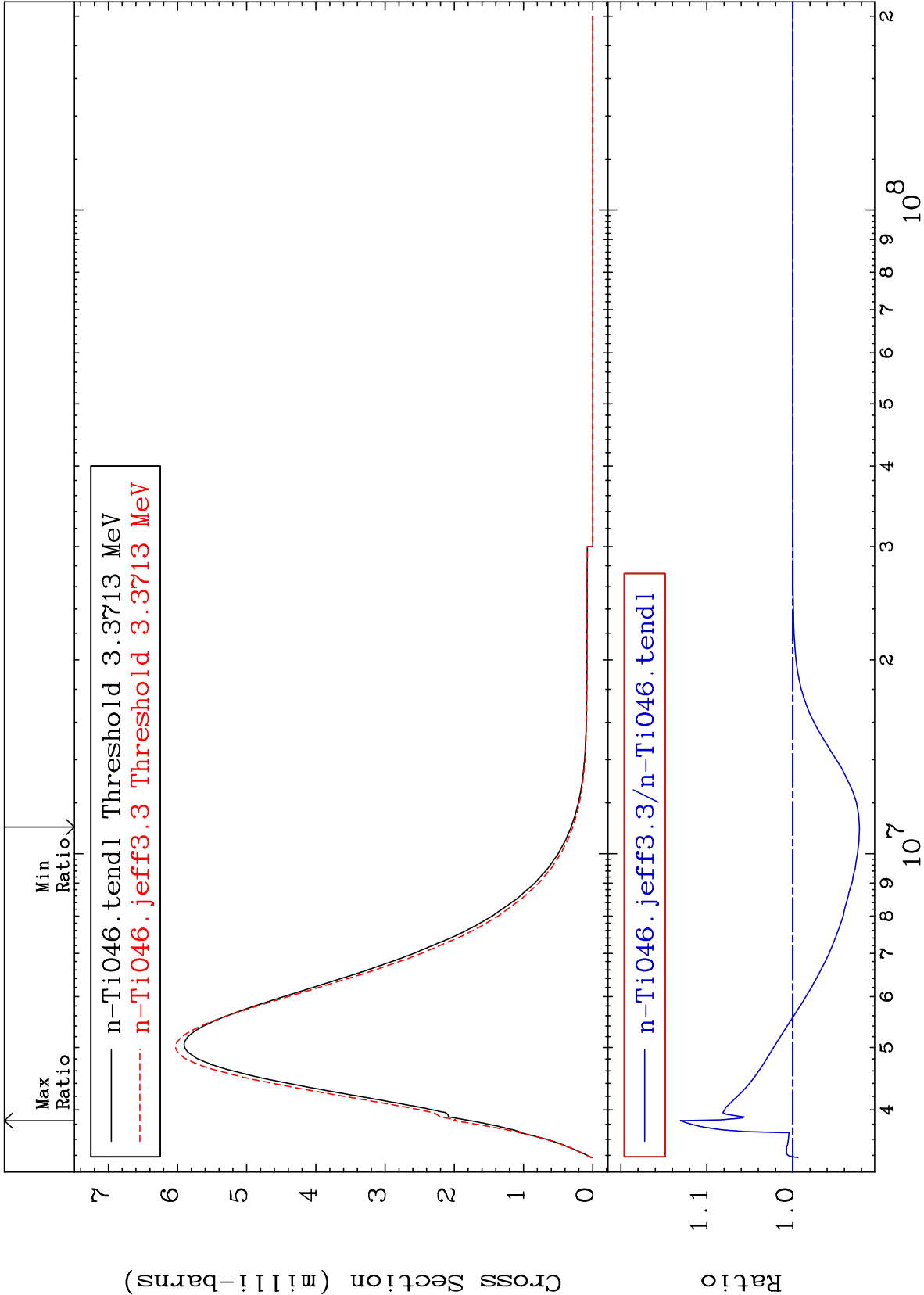
22

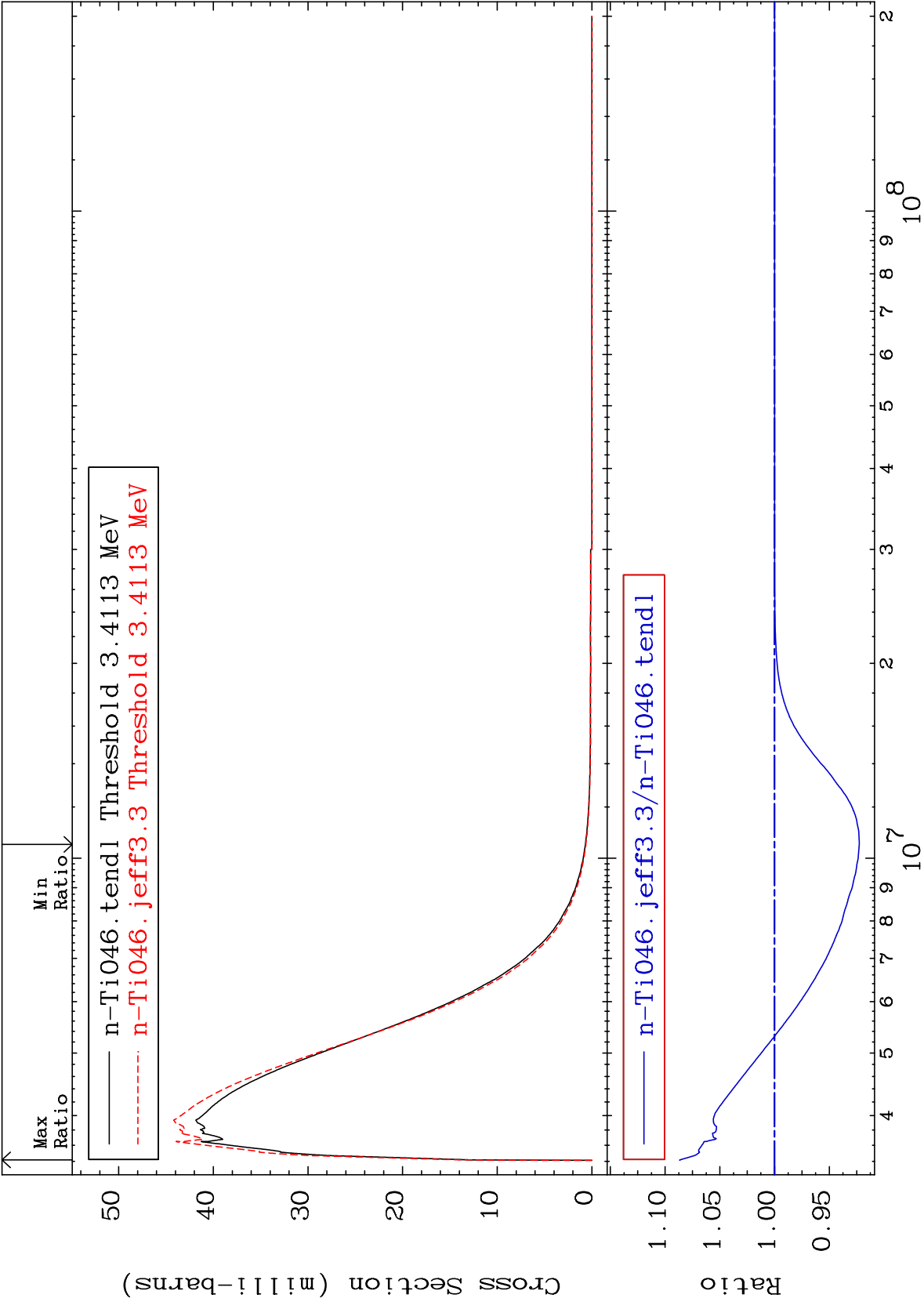
Incident Energy (eV)

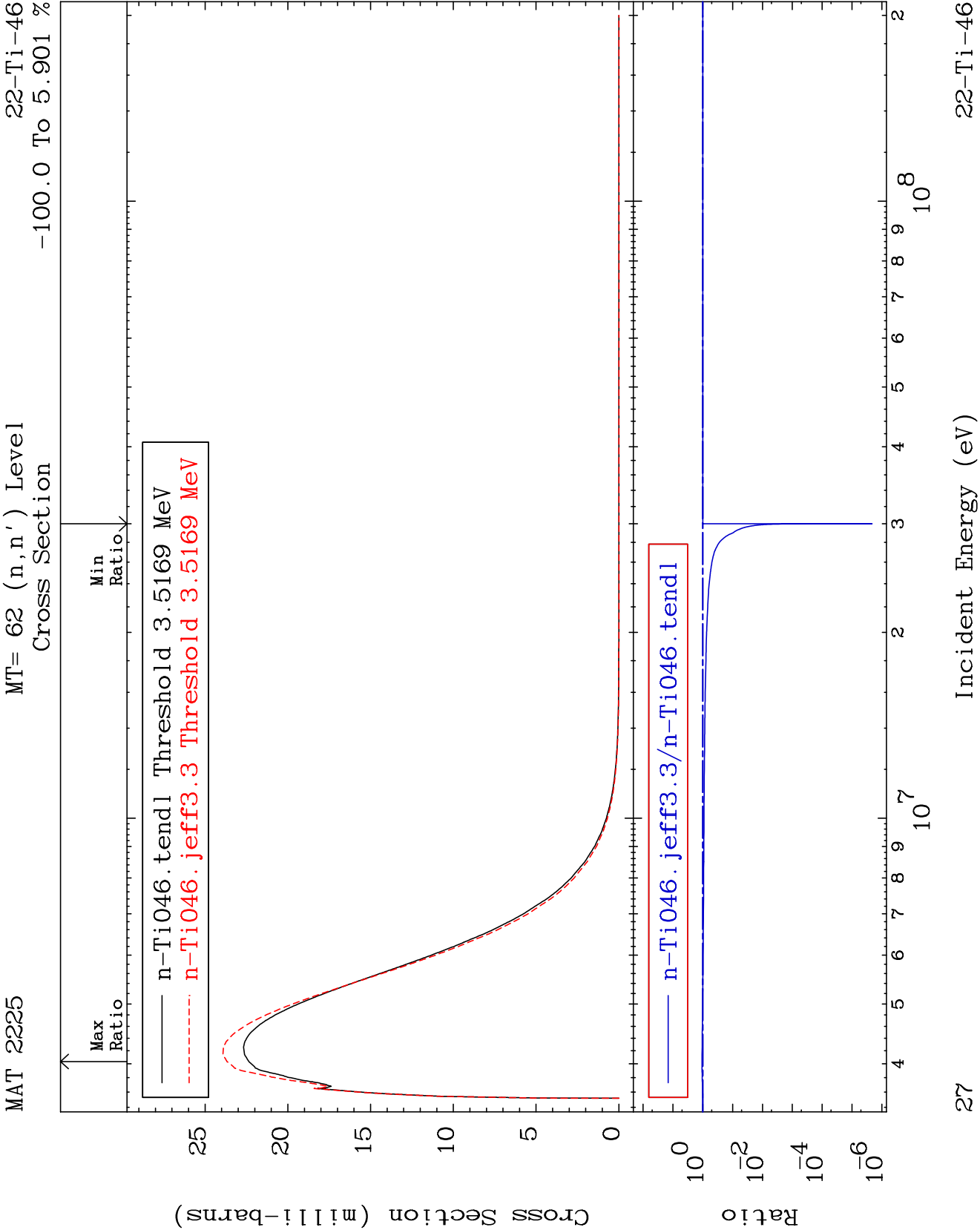
22-Ti-46

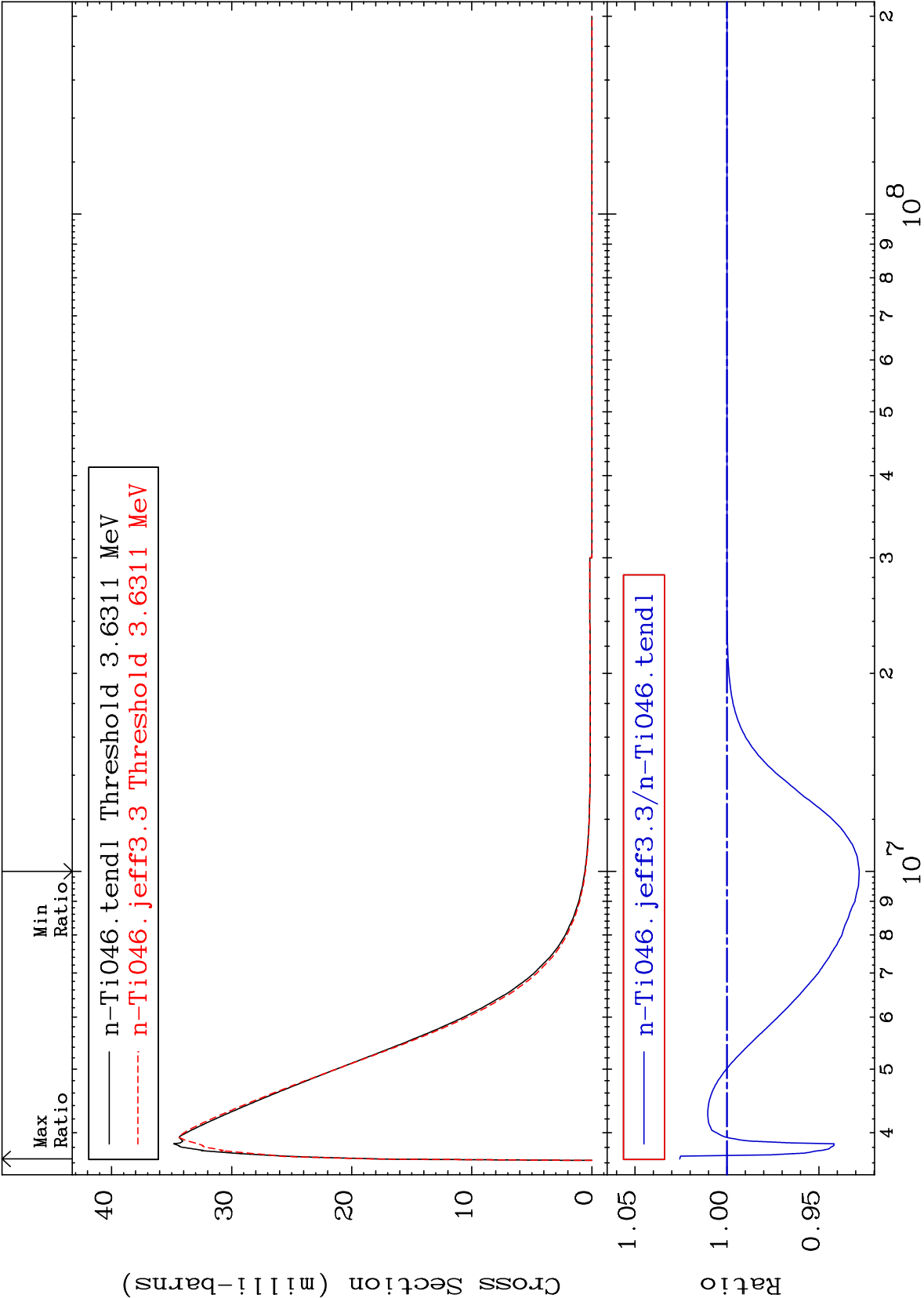


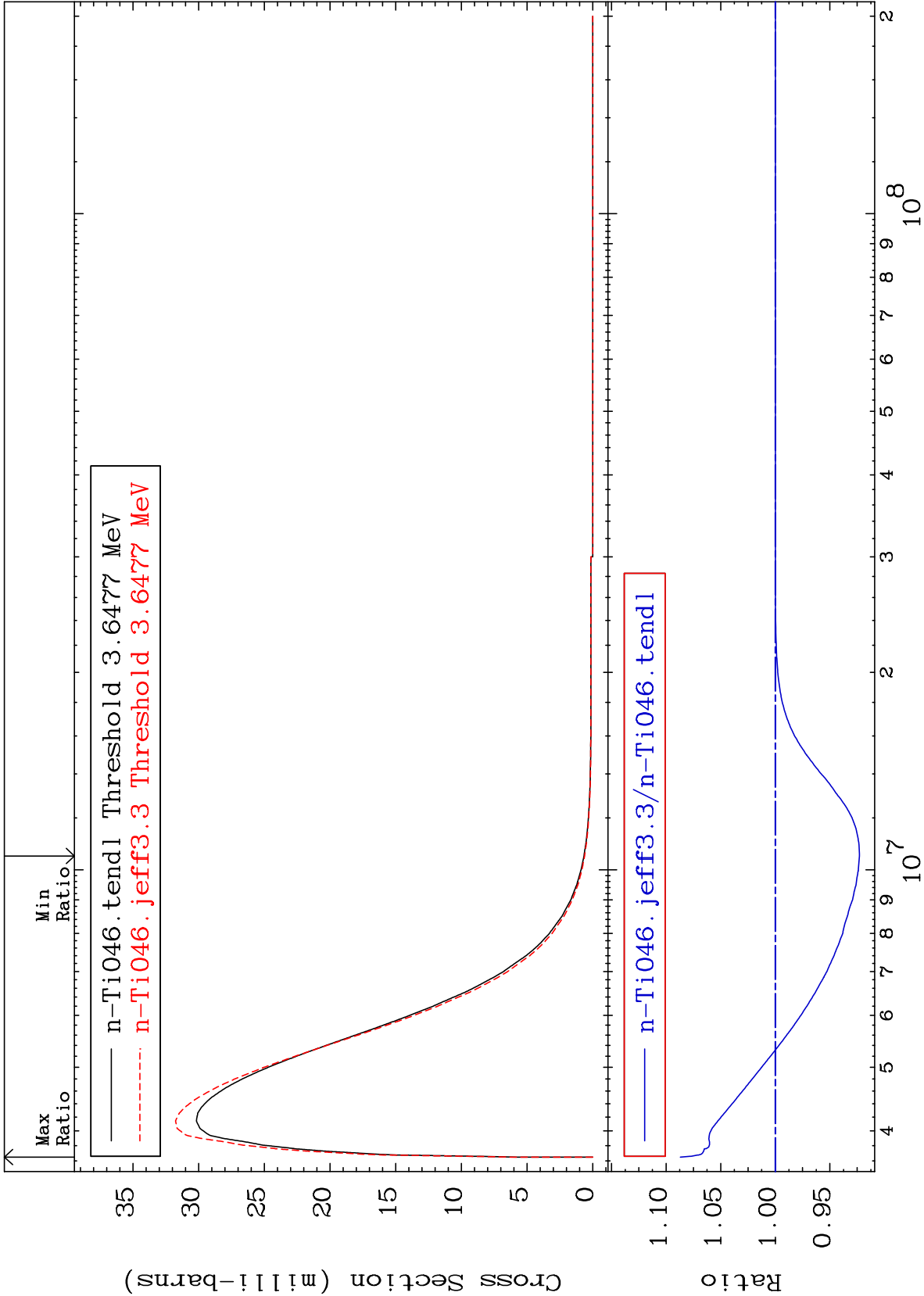












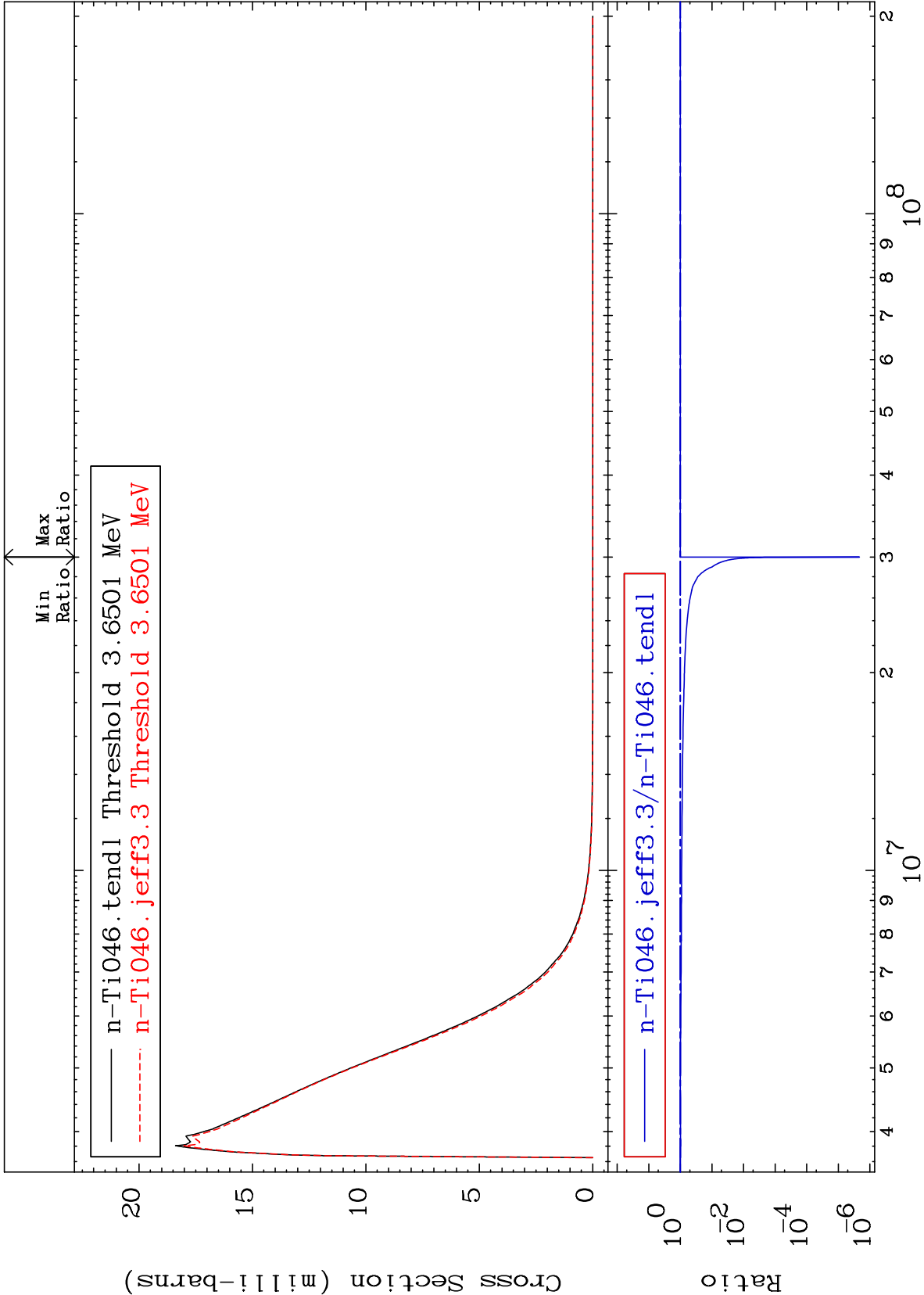
MAT 2225

MT= 65 (n,n') Level

22-Ti-46

-100.0 To 0.000 %

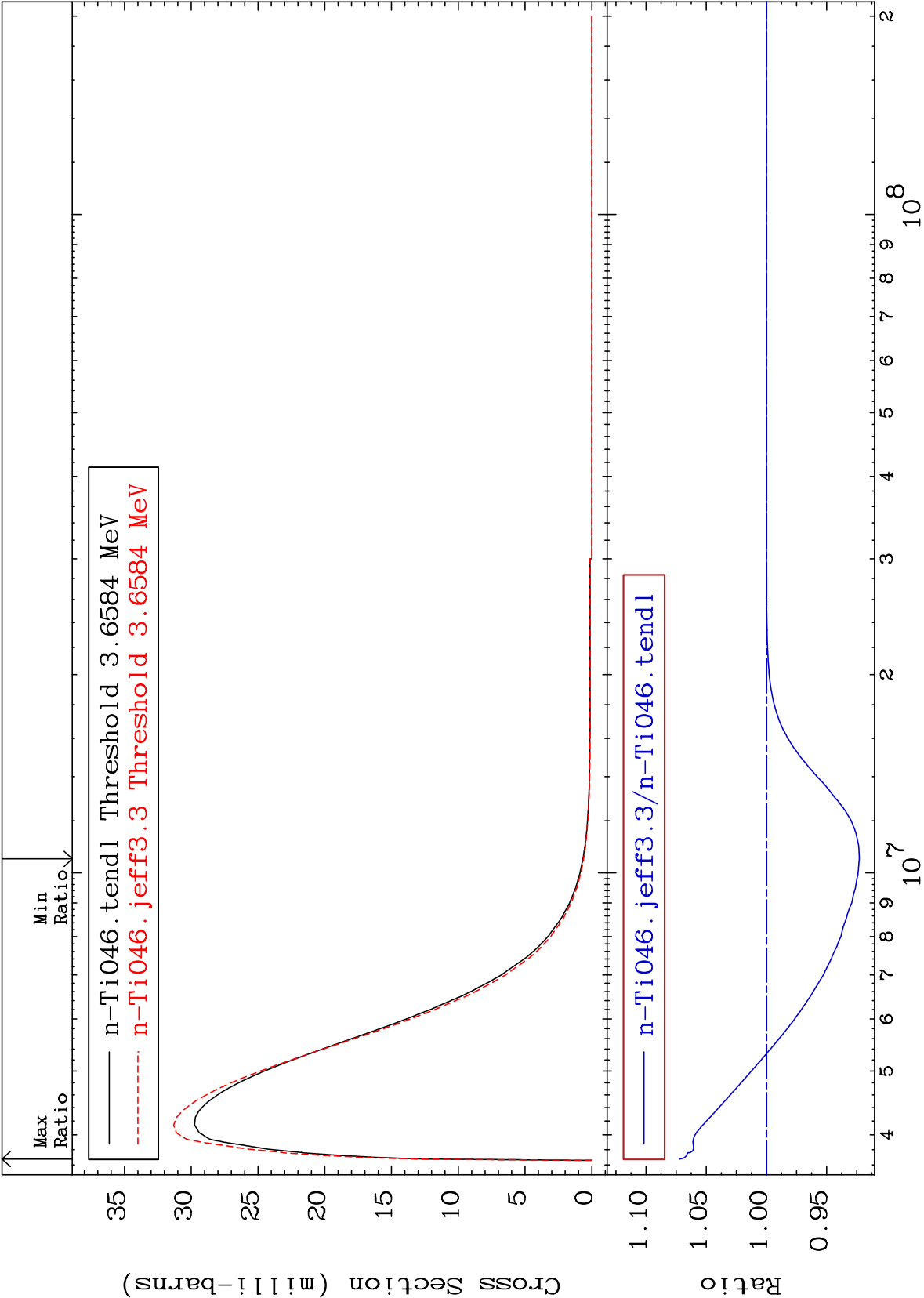
Cross Section

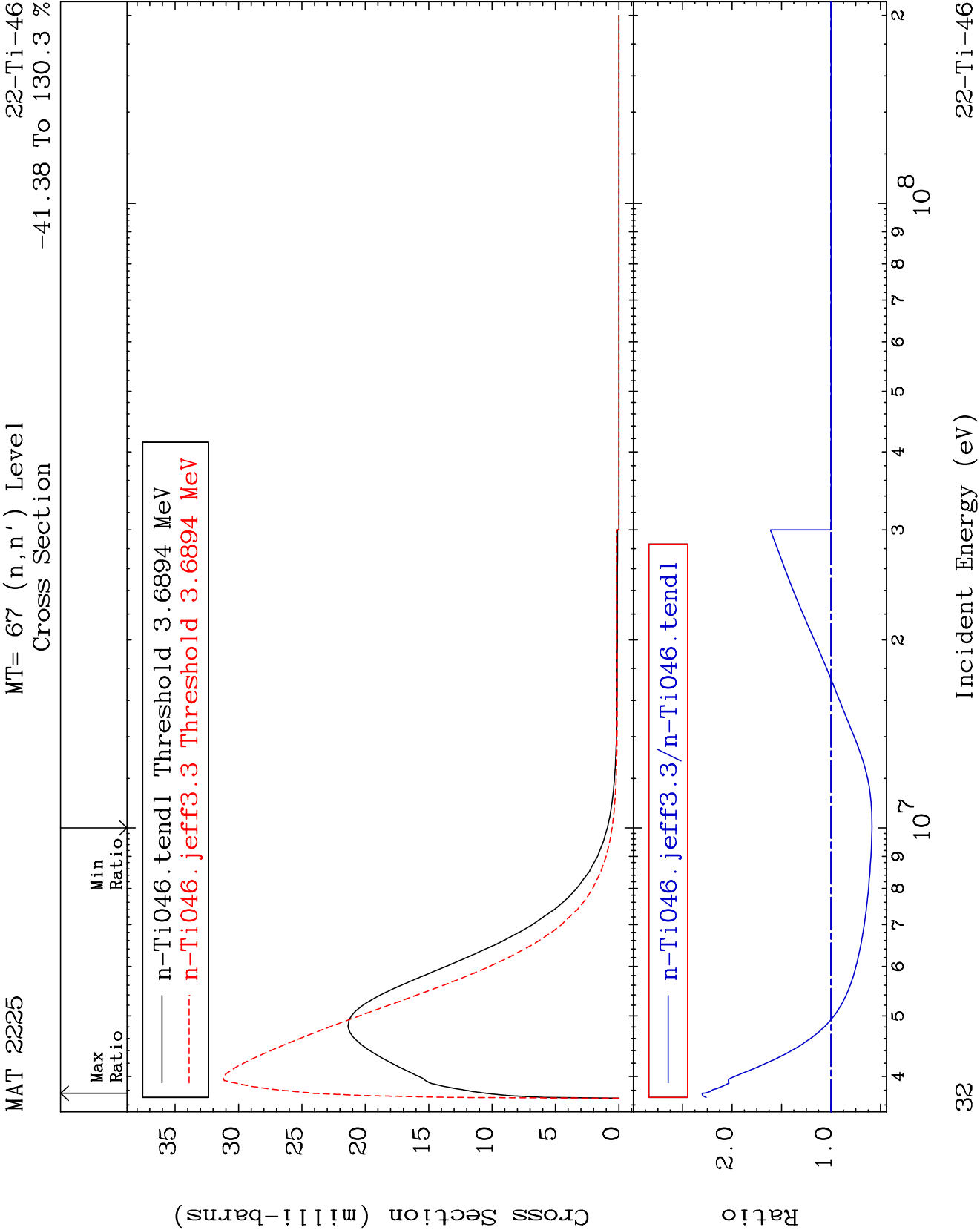


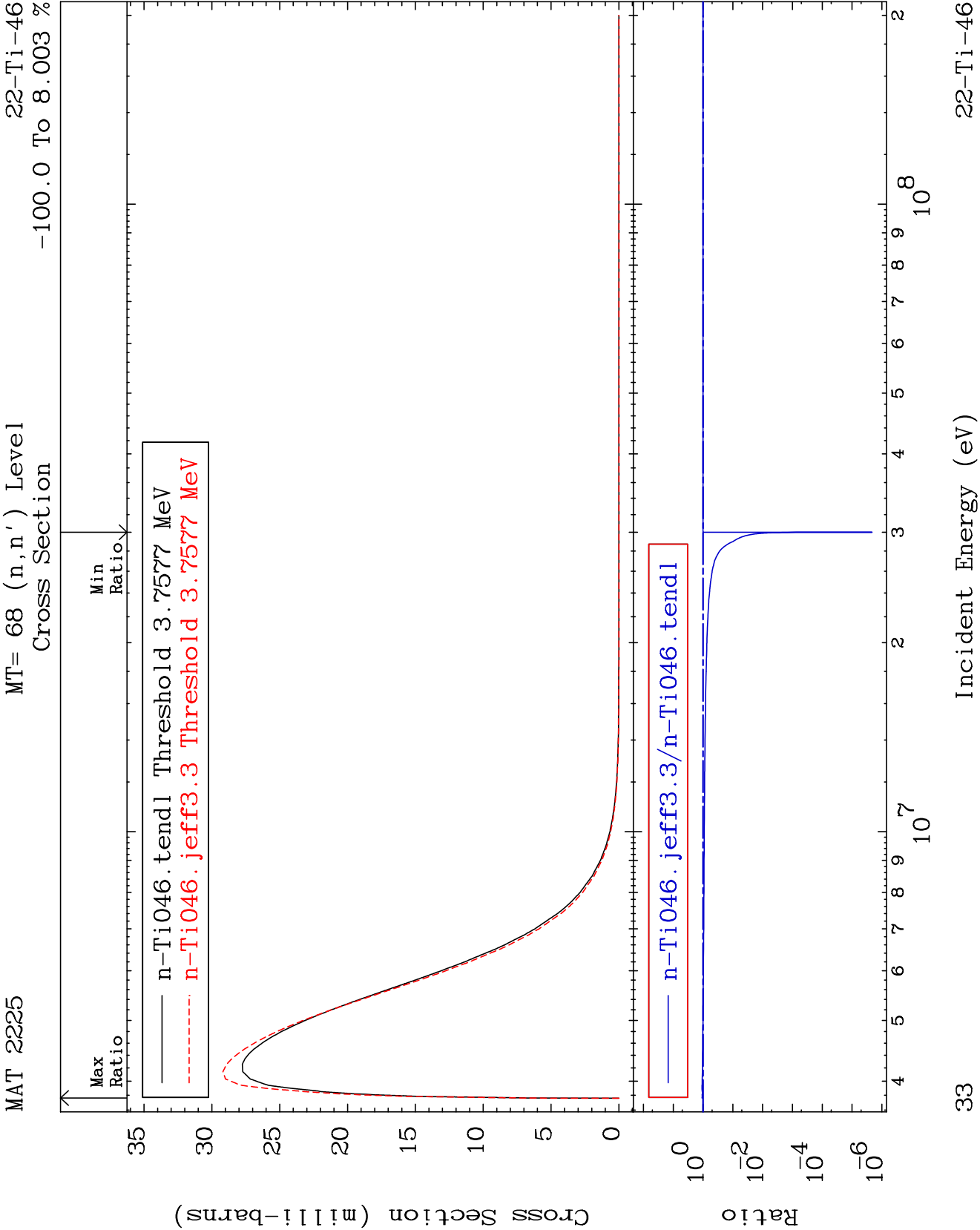
30

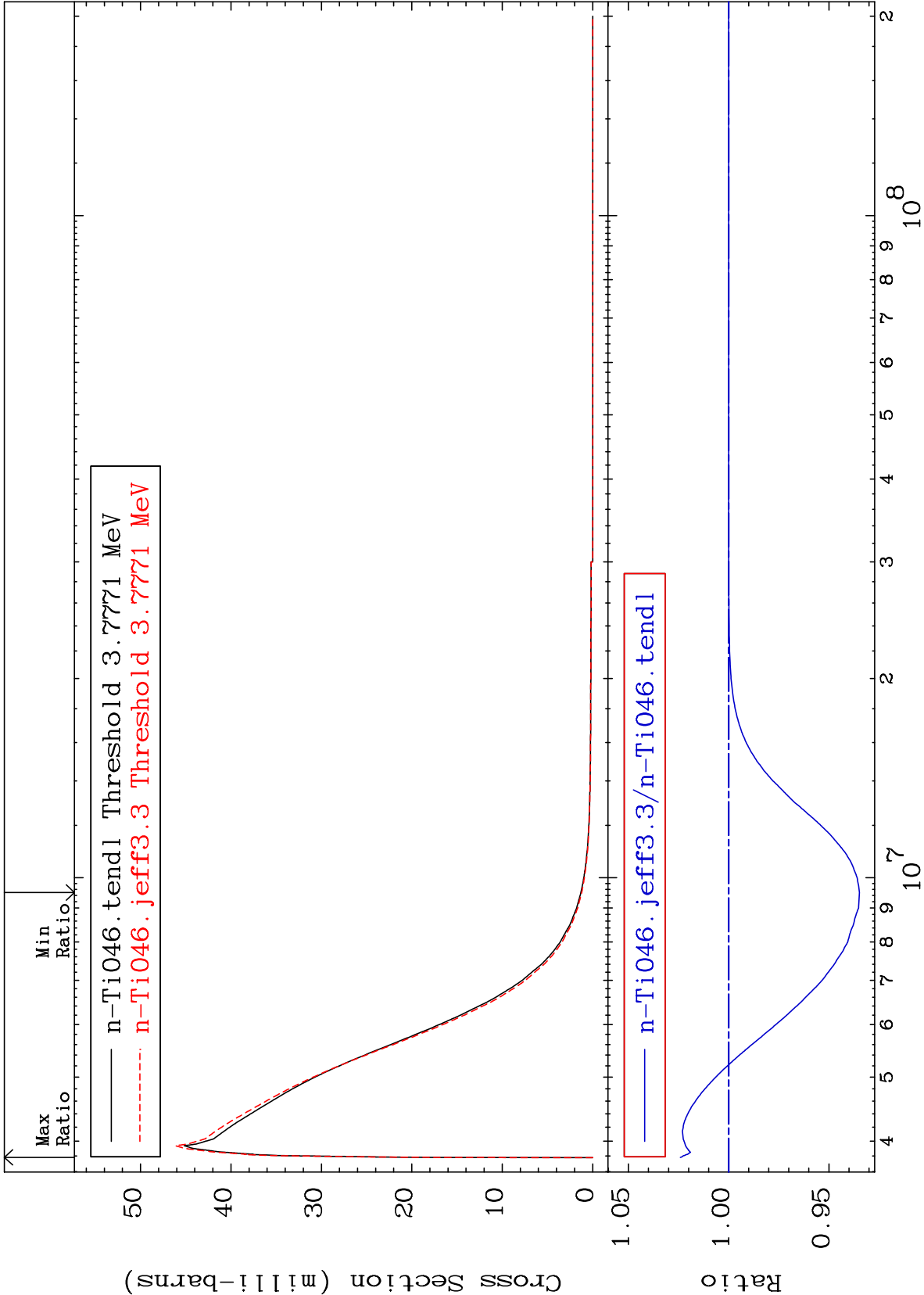
Incident Energy (eV)

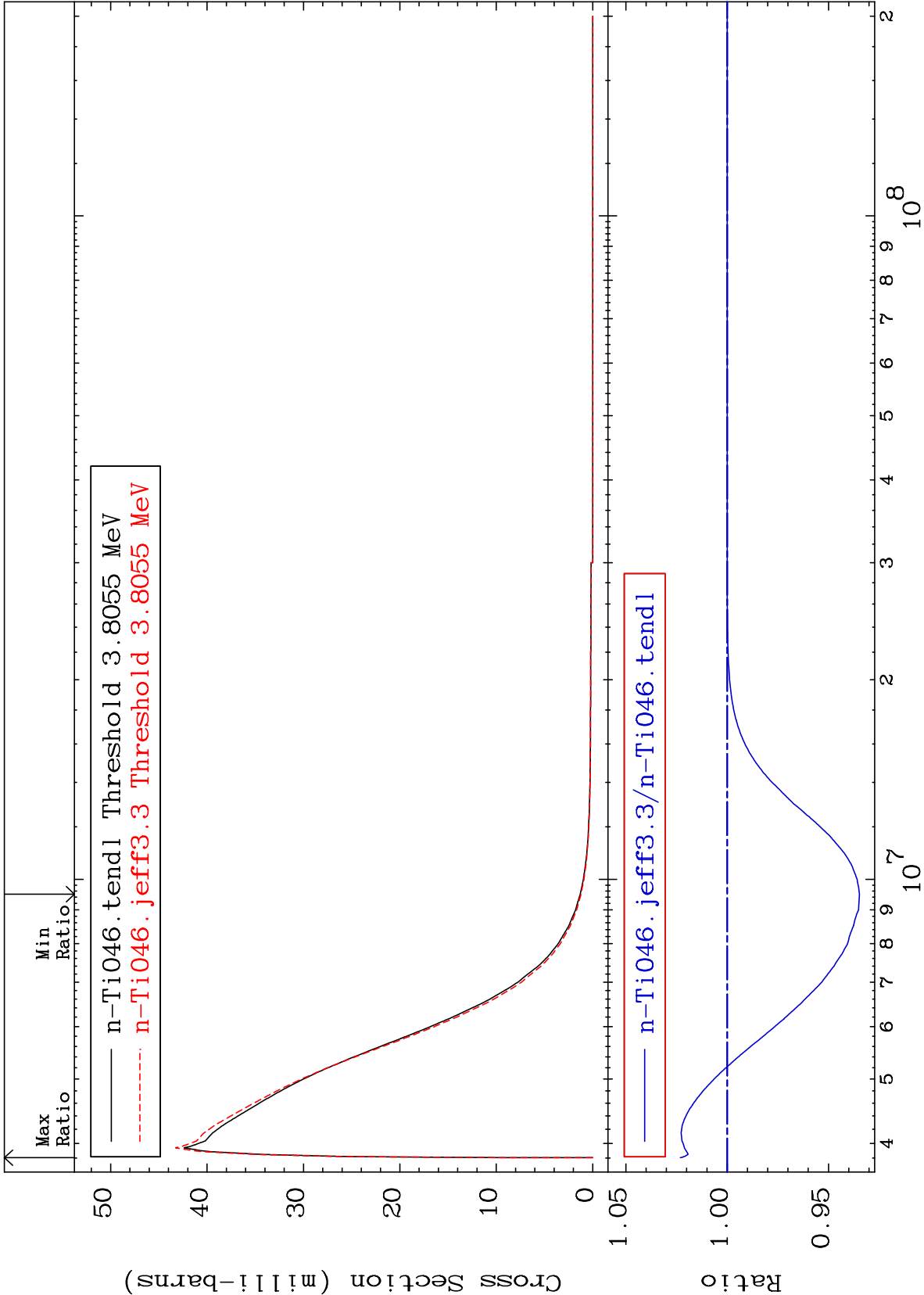
22-Ti-46

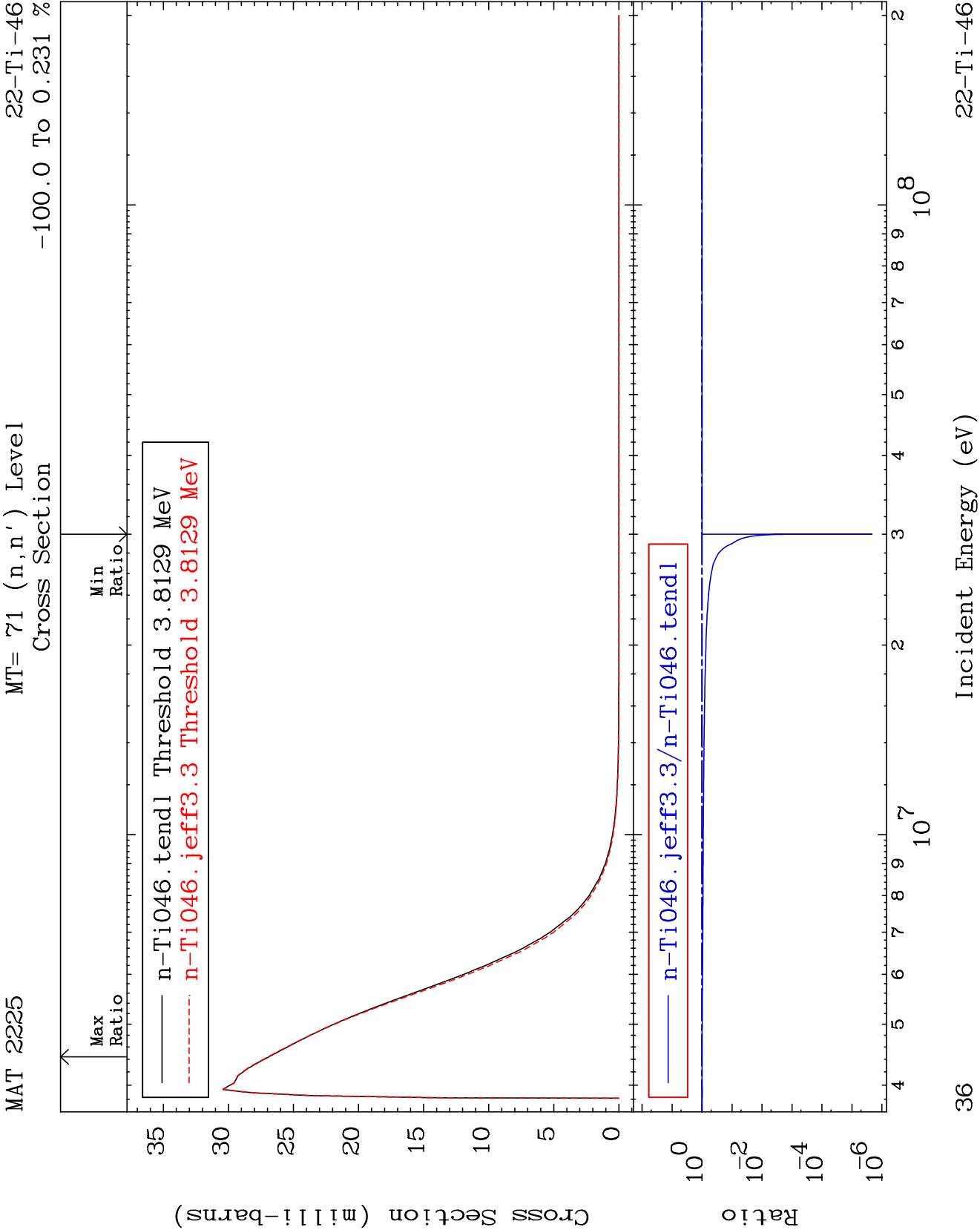








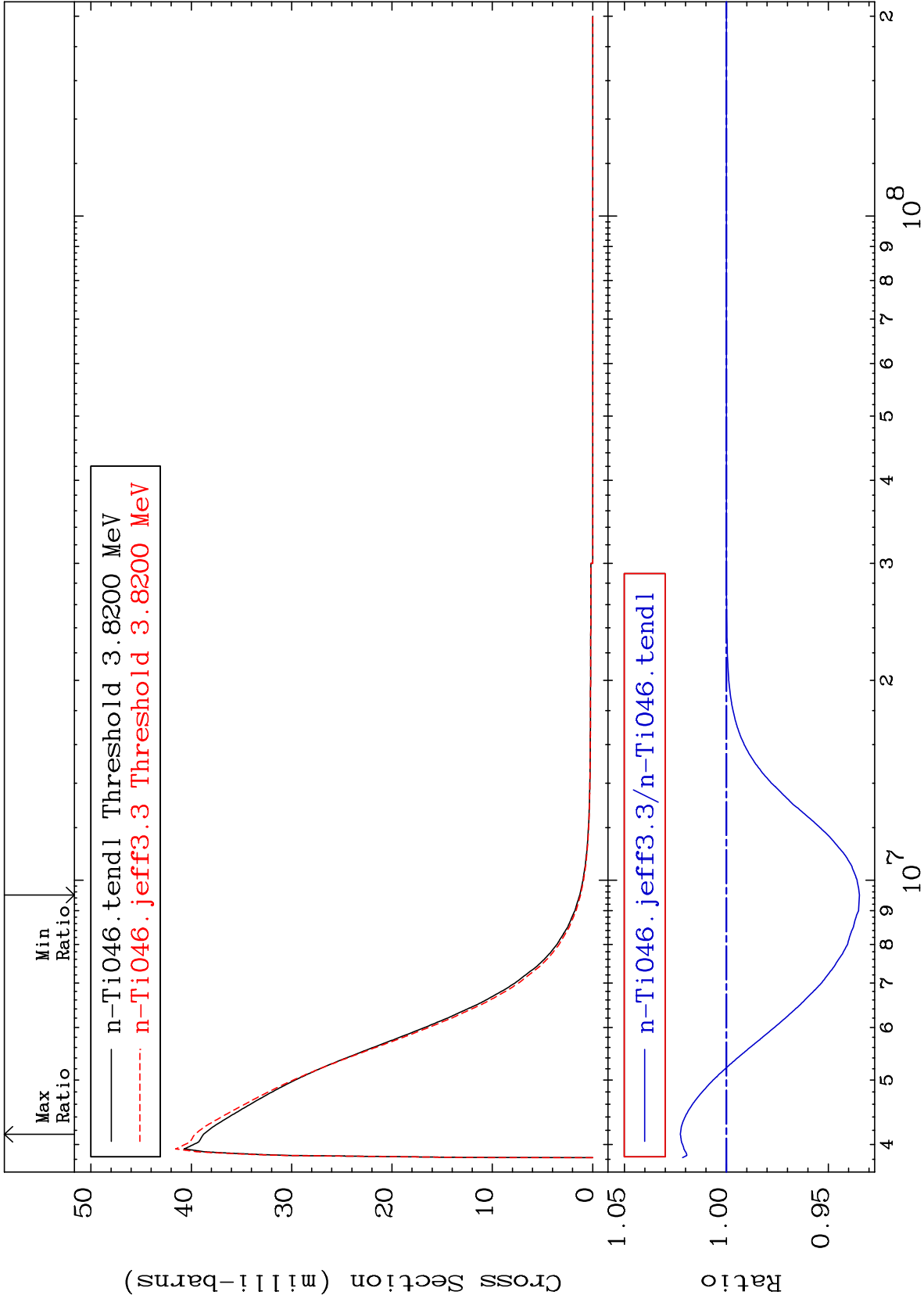




MAT 2225

MT= 72 (n,n') Level
Cross Section

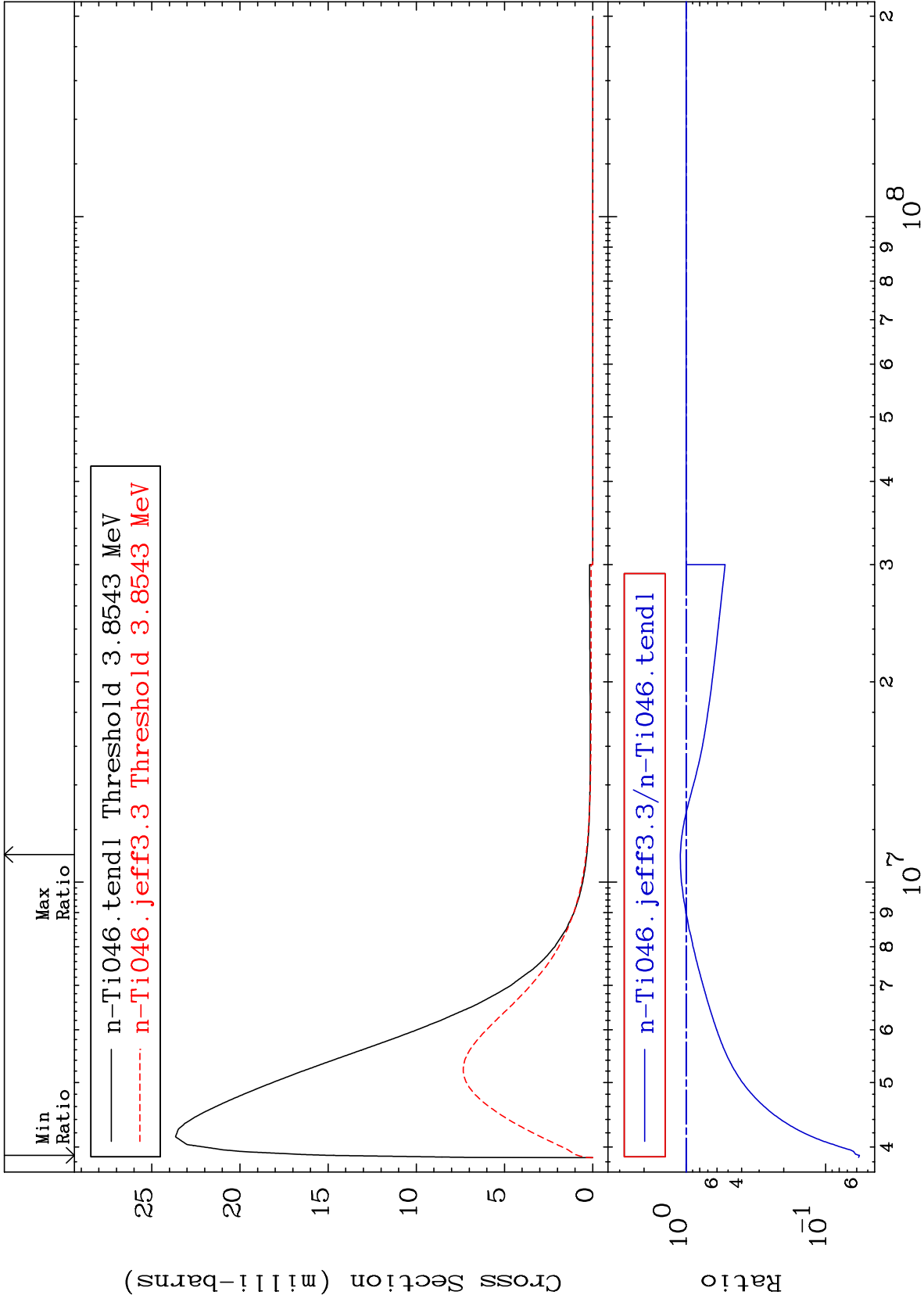
22-Ti-46
-6.527 To 2.258 %

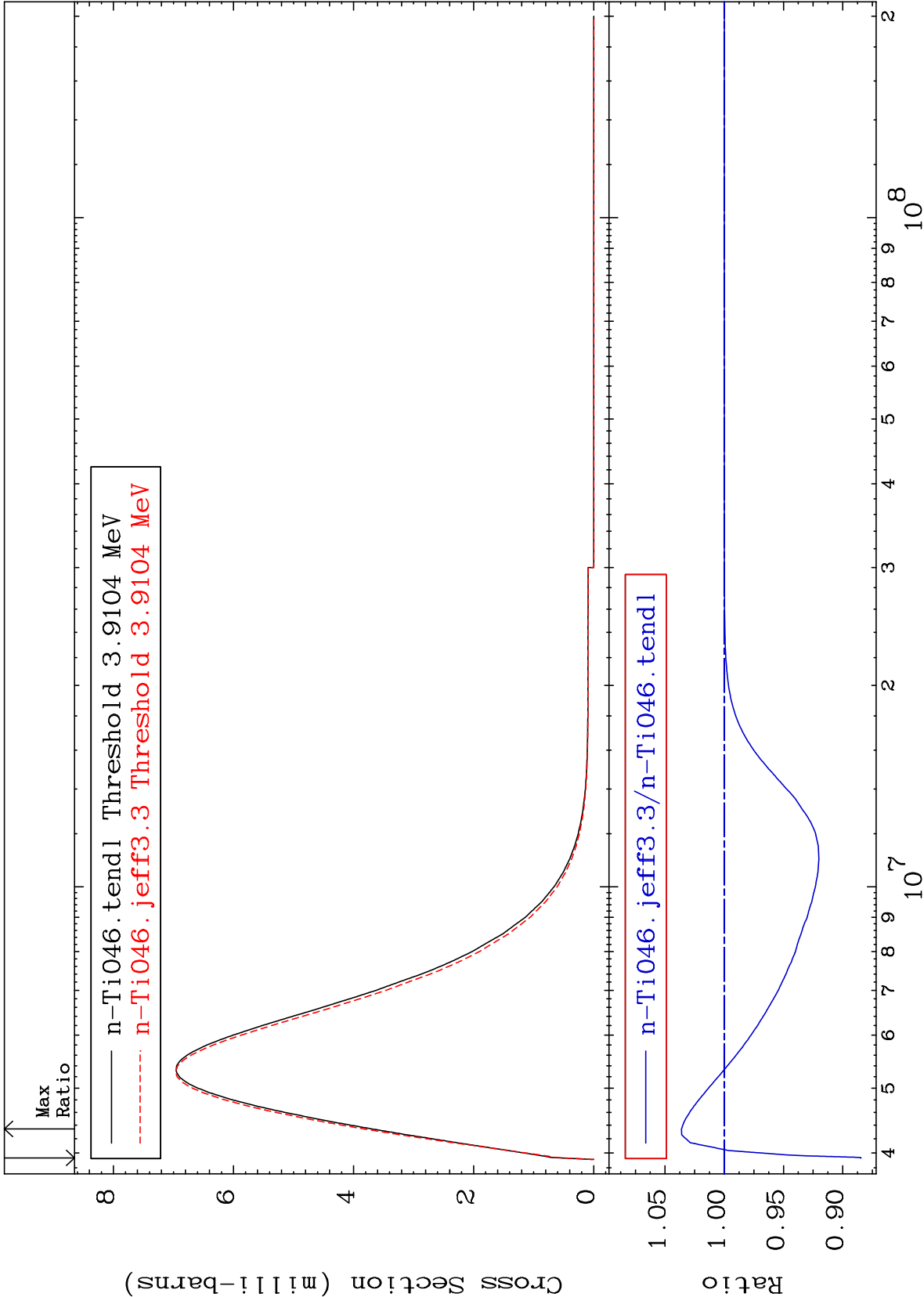


MAT 2225

MT= 73 (n,n') Level
Cross Section

22-Ti-46
-94.27 To 10.00 %

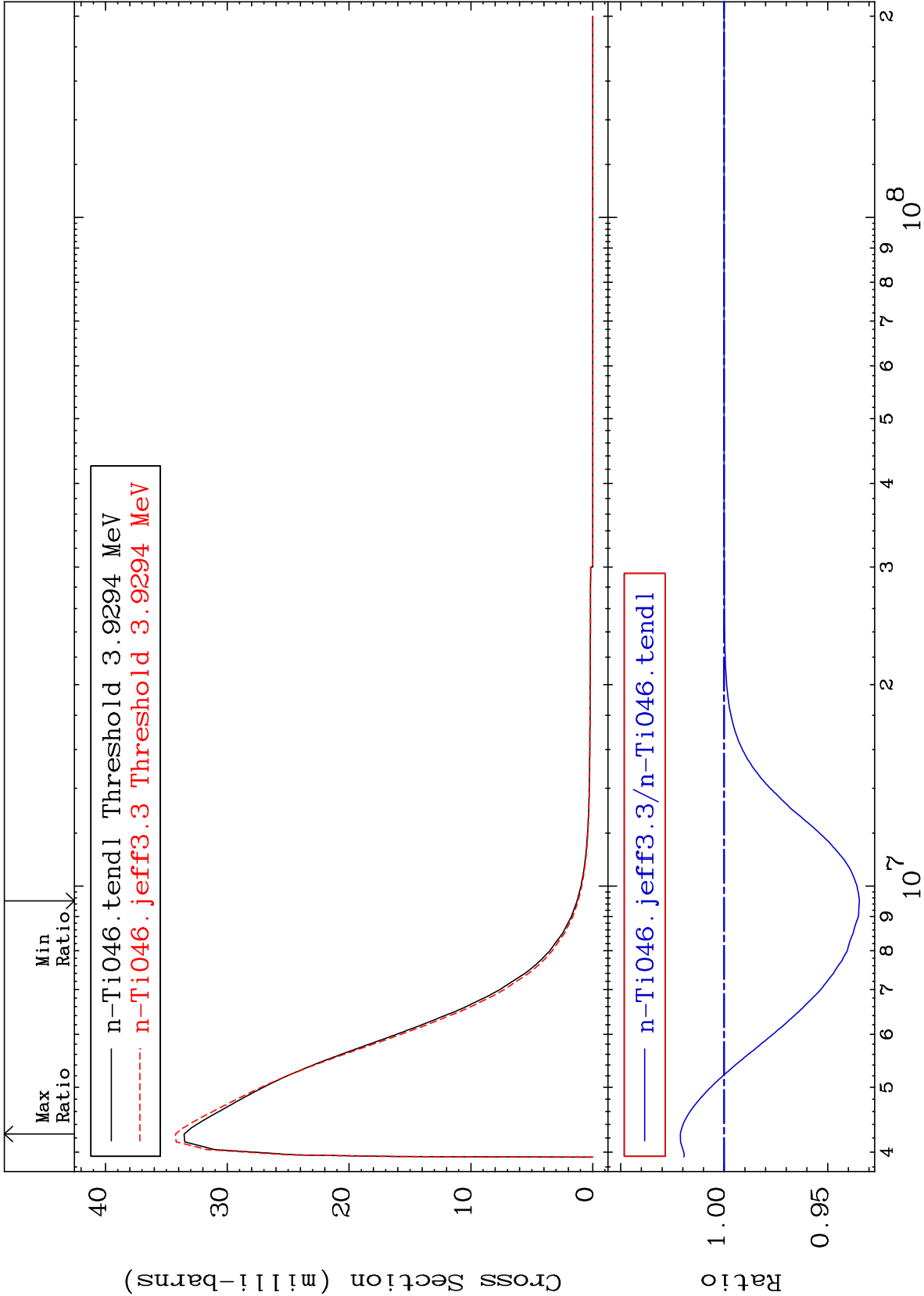




MAT 2225

MT= 75 (n,n') Level
Cross Section

22-Ti-46
-6.535 To 2.118 %



40

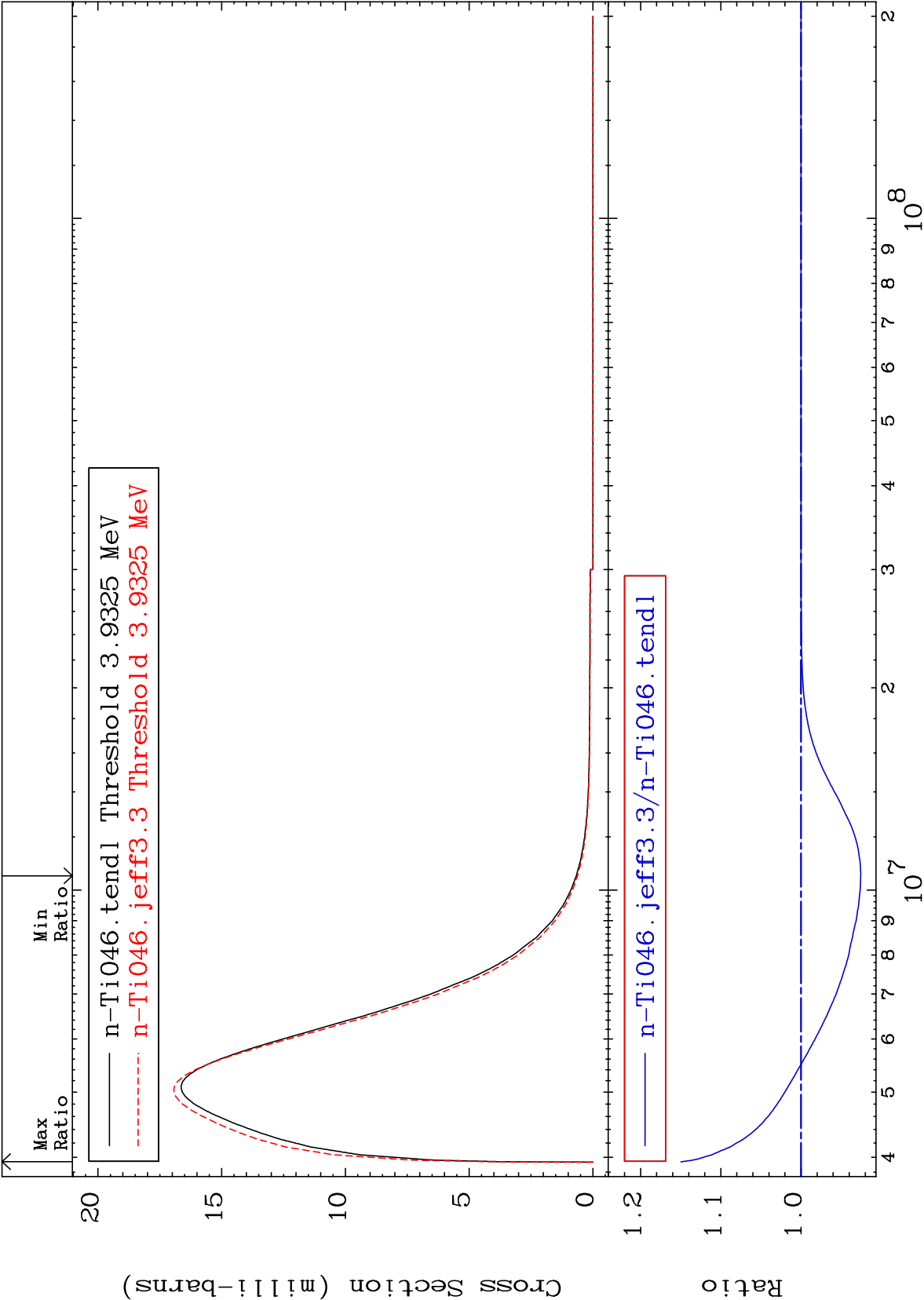
Incident Energy (eV)

22-Ti-46

MAT 2225

MT= 76 (n,n') Level
Cross Section

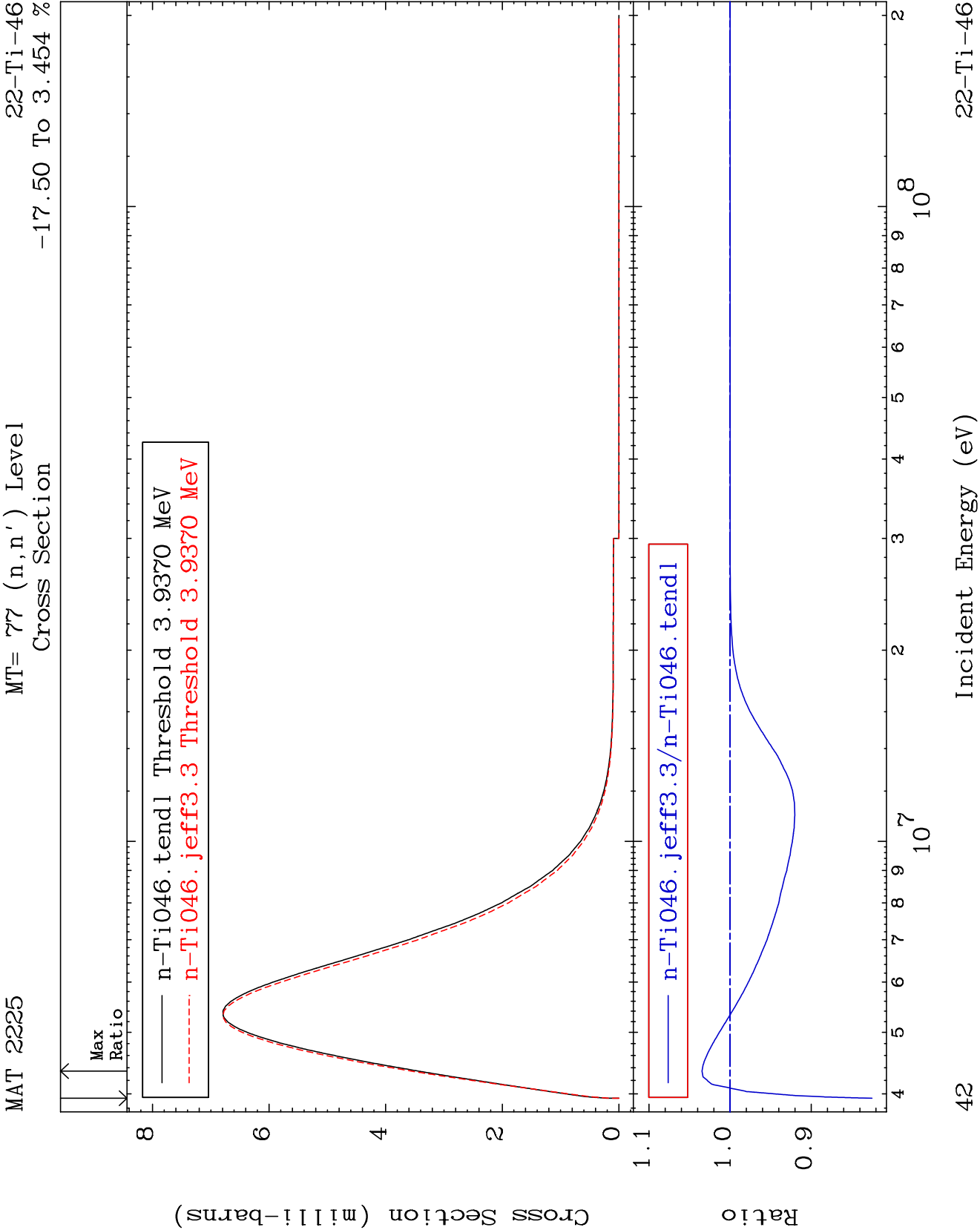
22-Ti-46
-7.436 To 14.97 %



41

Incident Energy (eV)

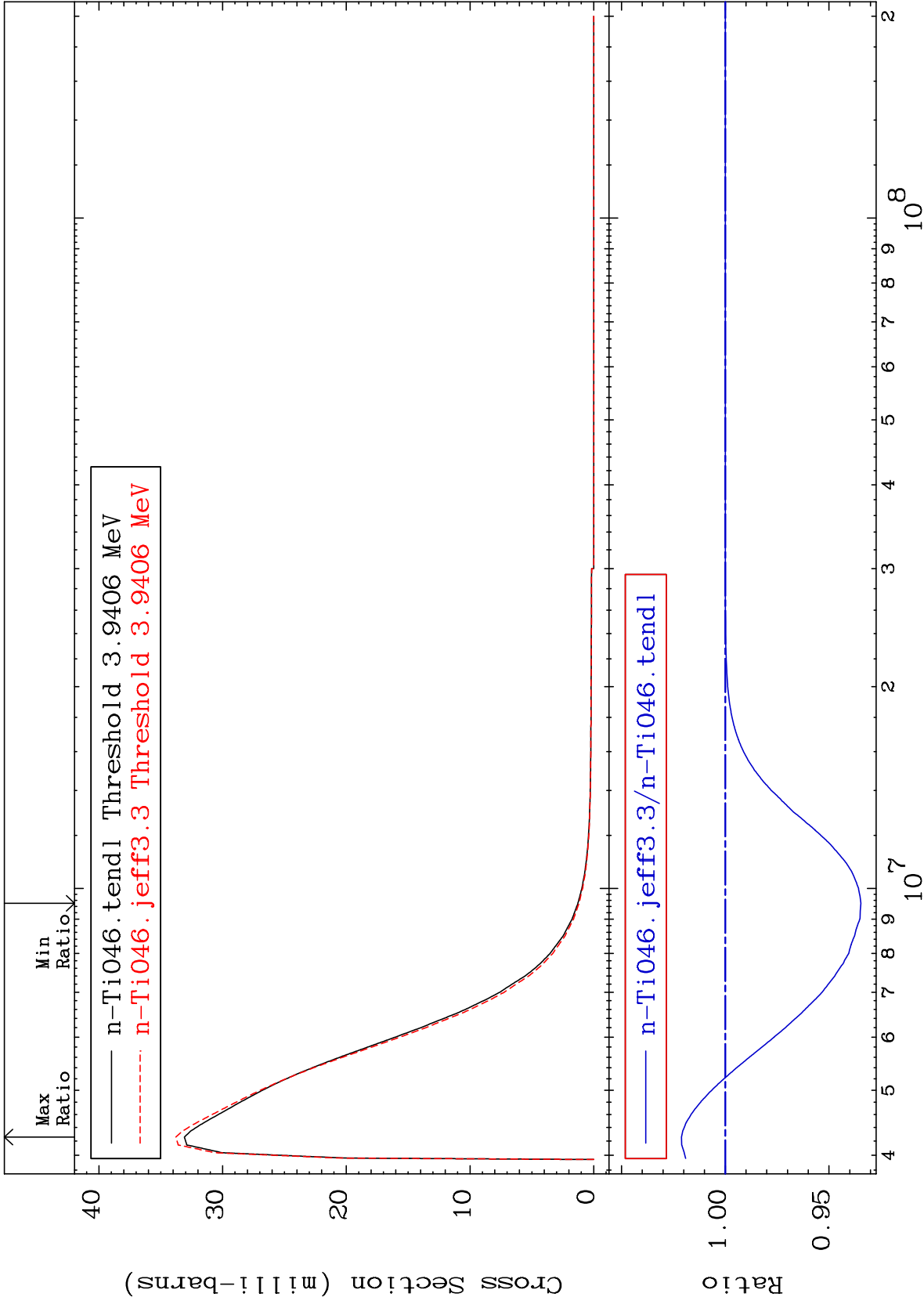
22-Ti-46



MAT 2225

MT= 78 (n,n') Level
Cross Section

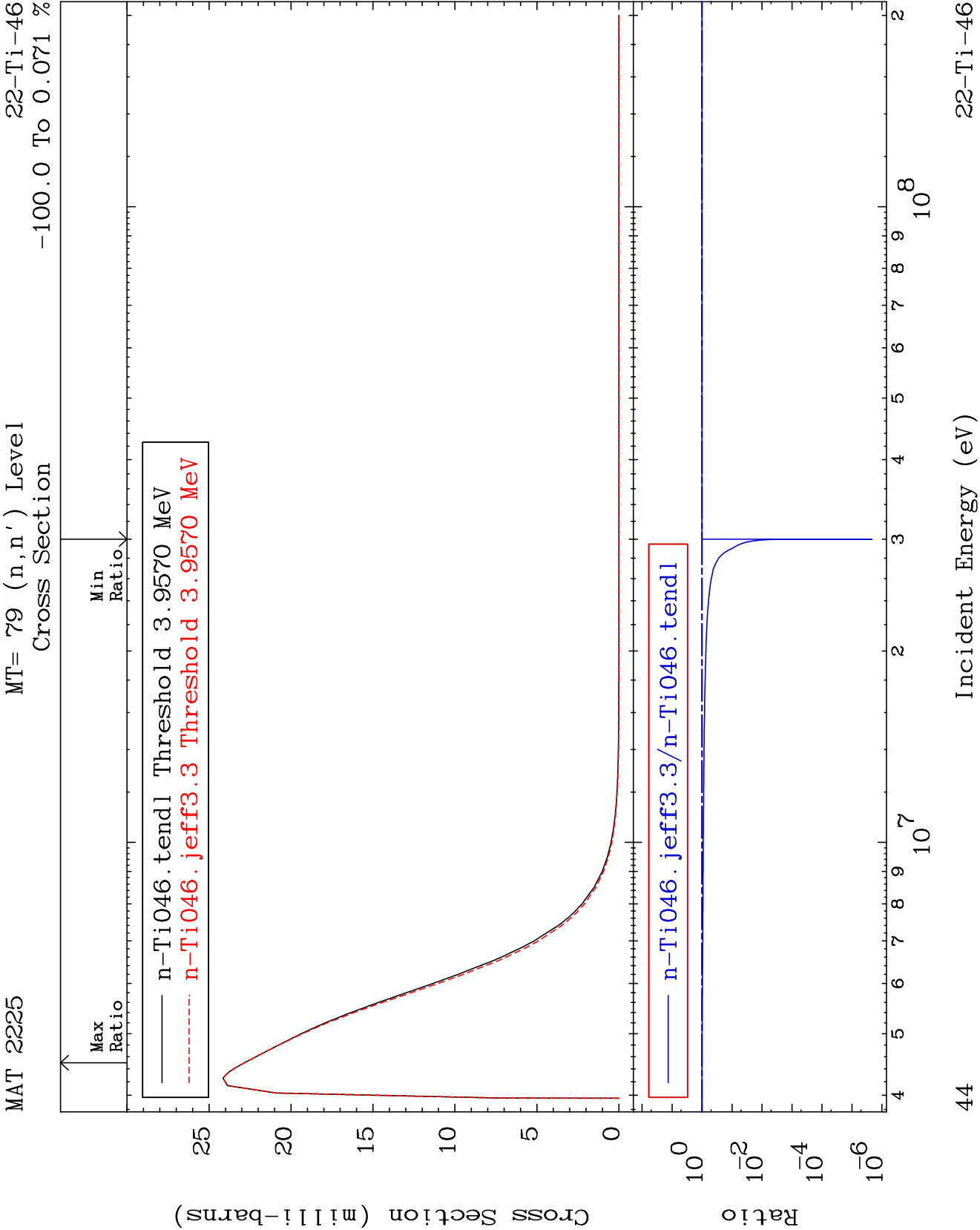
22-Ti-46
-6.536 To 2.108 %



43

Incident Energy (eV)

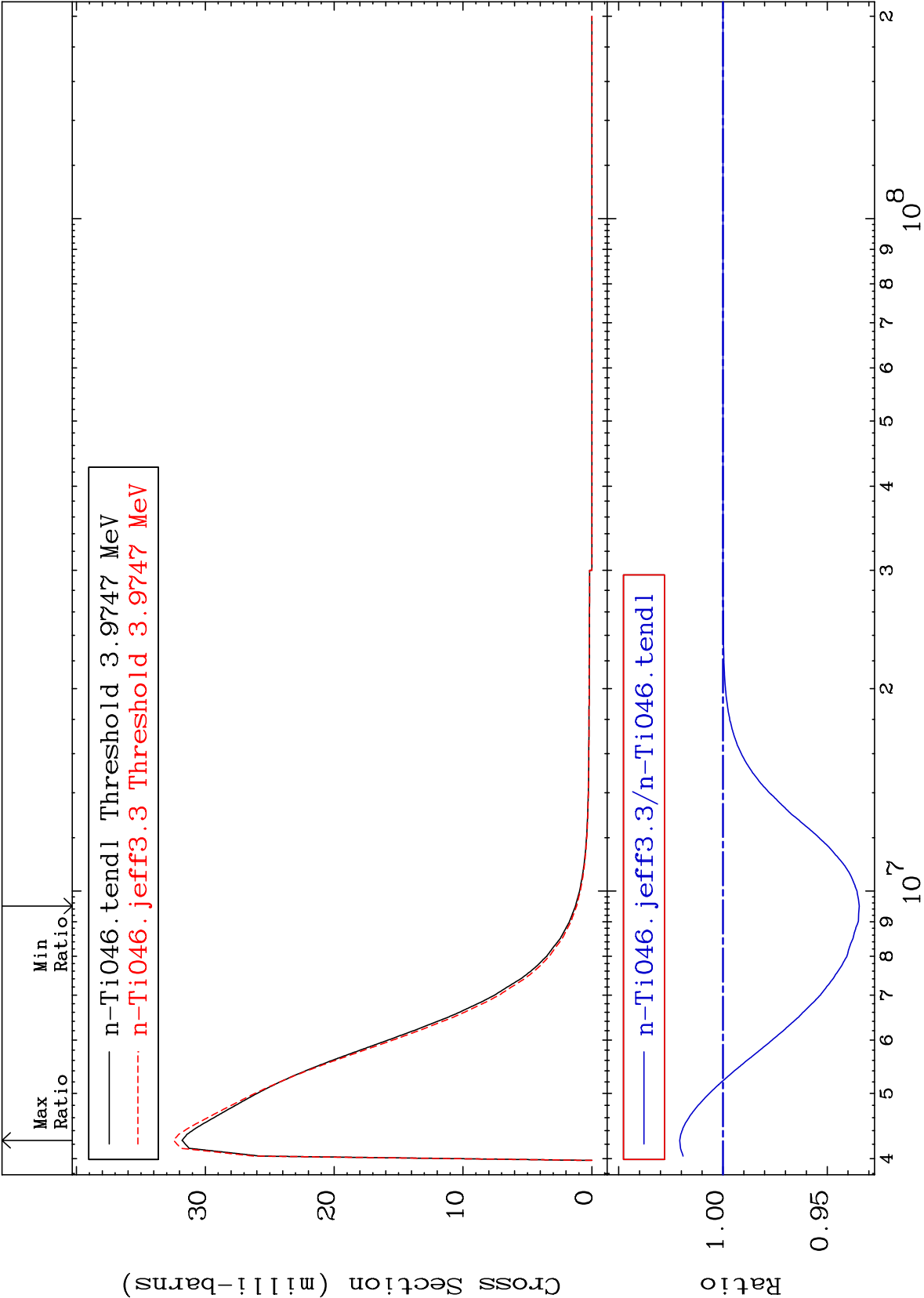
22-Ti-46



MAT 2225

MT= 80 (n,n') Level
Cross Section

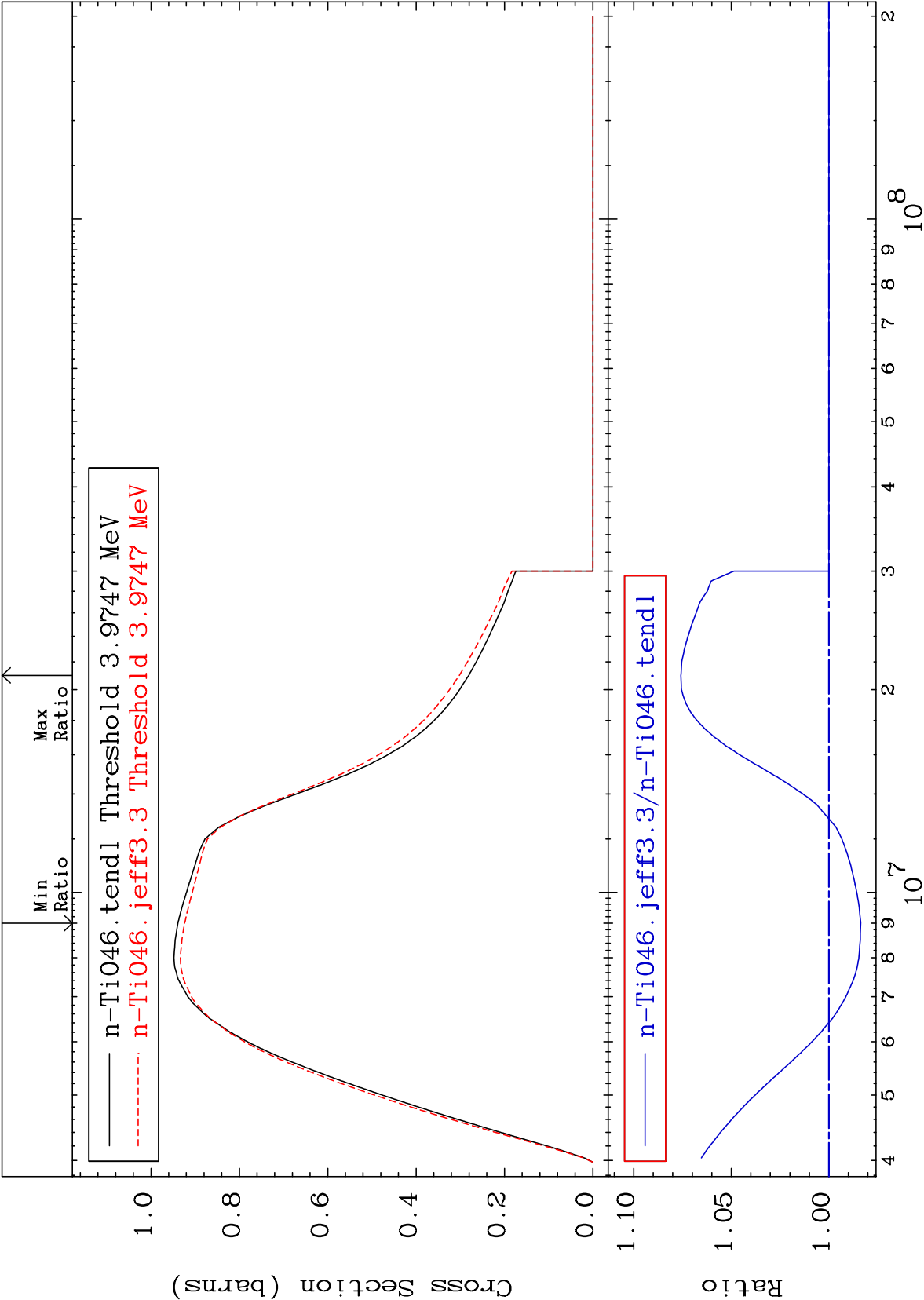
22-Ti-46
-6.538 To 2.073 %

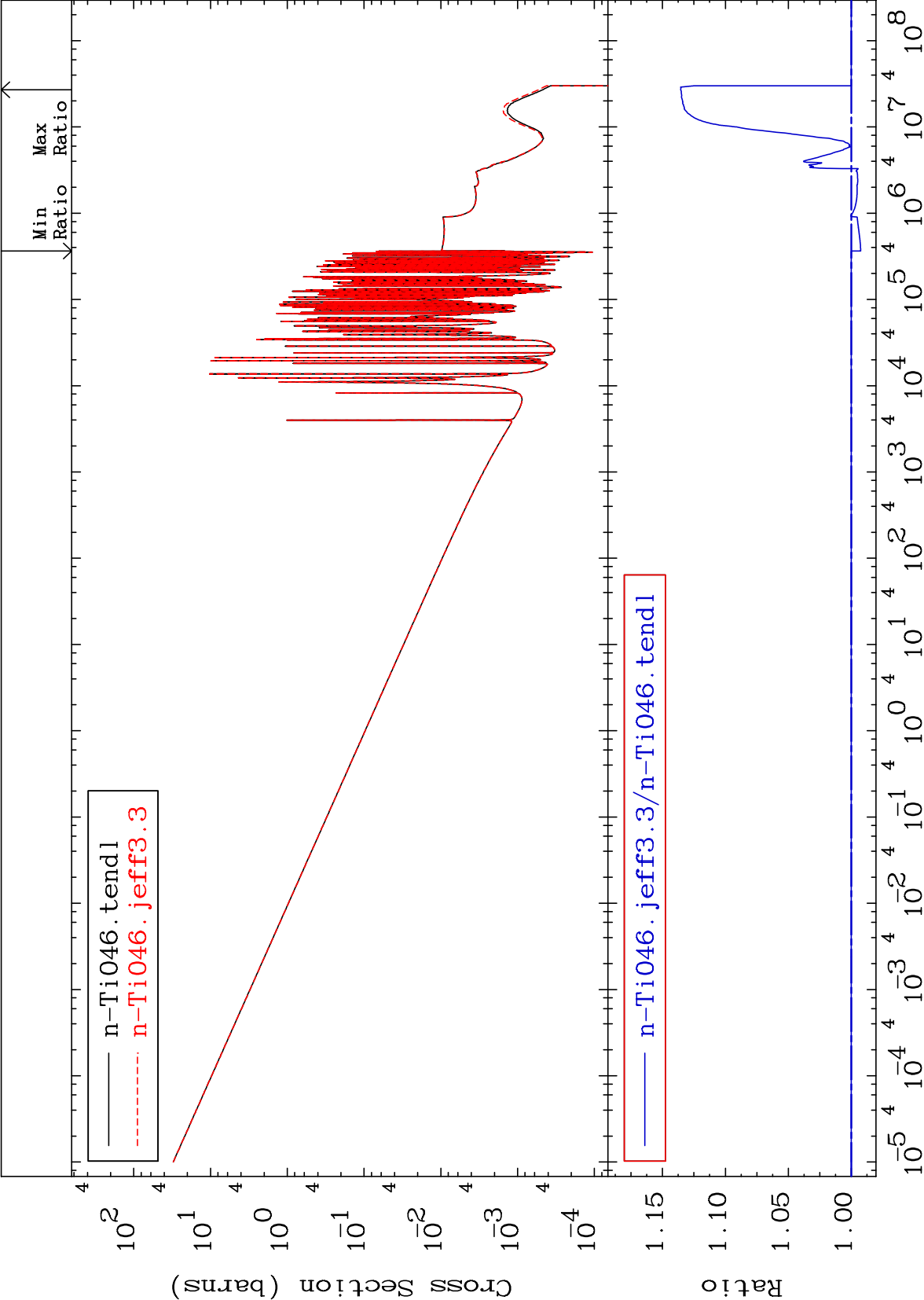


45

Incident Energy (eV)

22-Ti-46



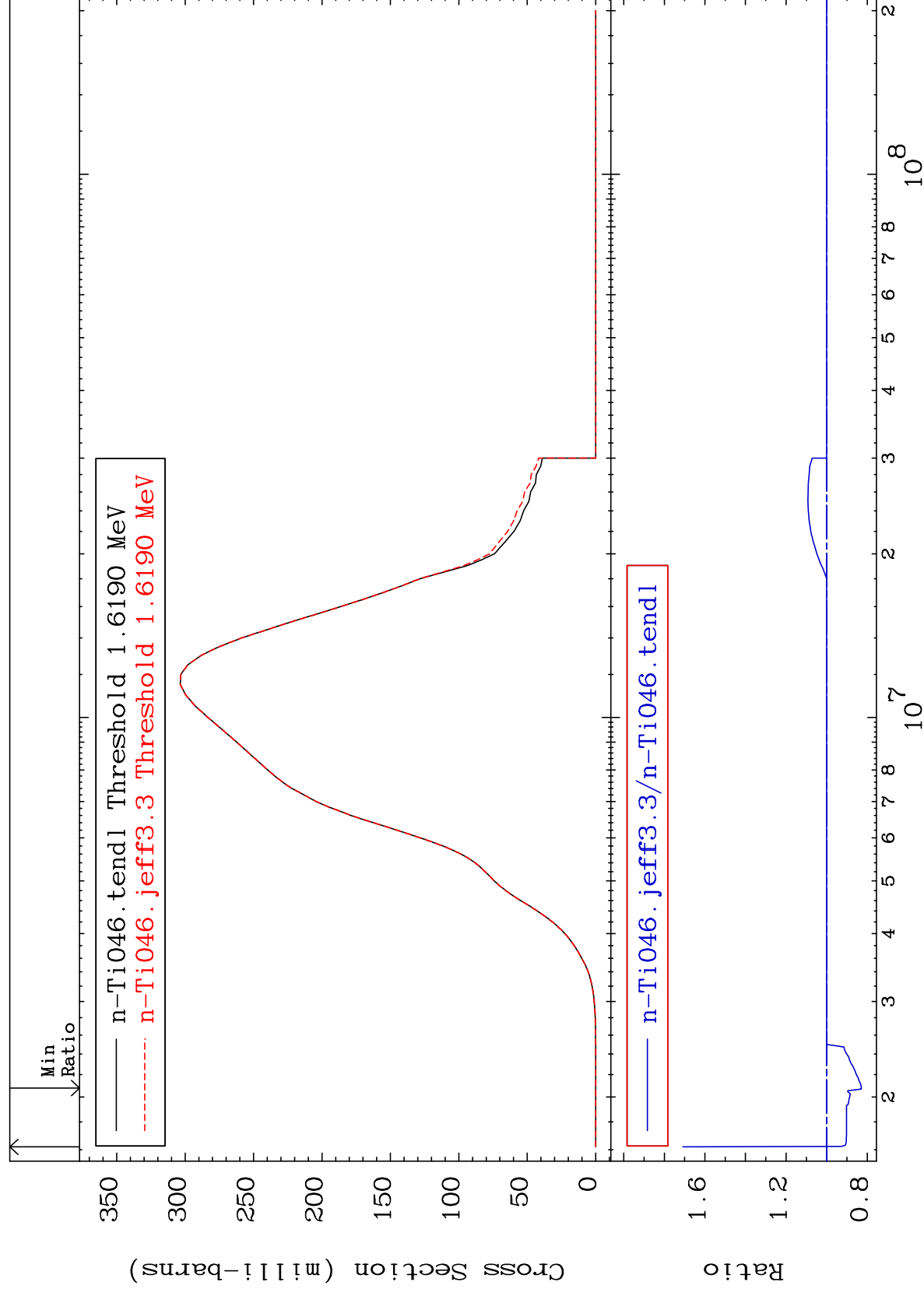


MAT 2225

$$(n, p)$$

Cross Section

22-Ti-46
-17.06 To 70.83 %



48

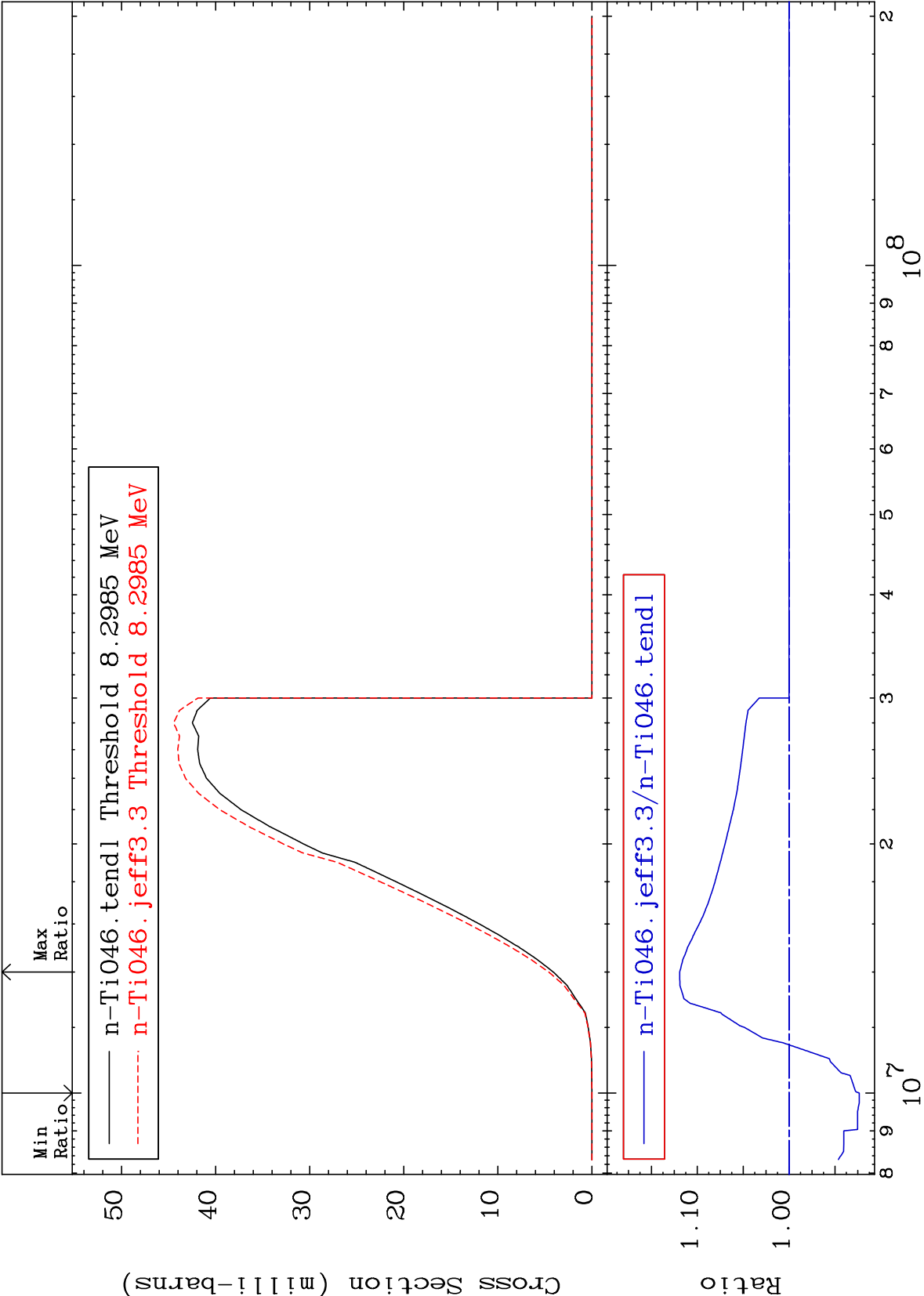
Incident Energy (eV)

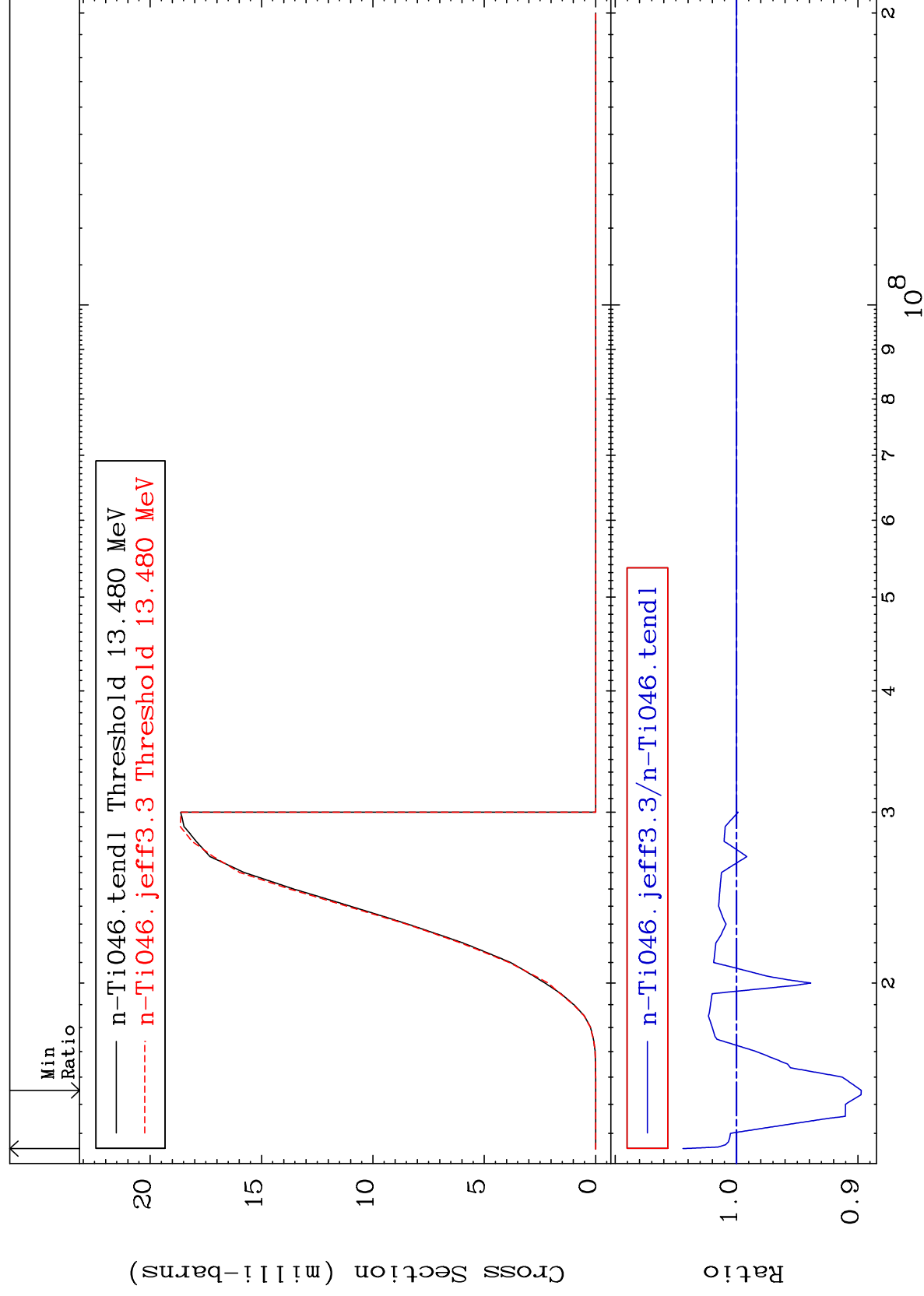
22-Ti-46

MAT 2225

(n,d)
Cross Section

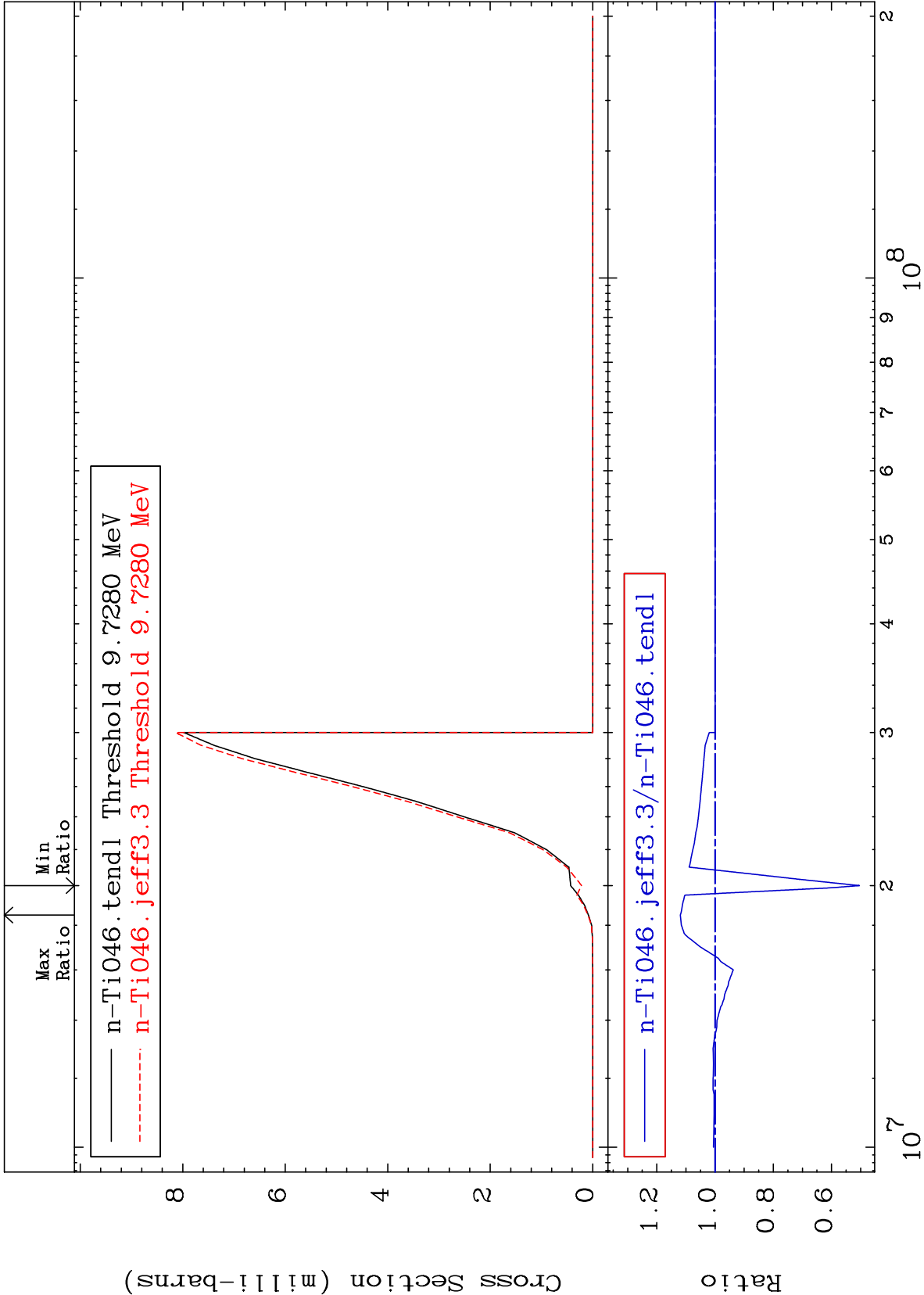
22-Ti-46
-7.652 To 11.91 %





Cross Section

-49.58 To 11.91 %



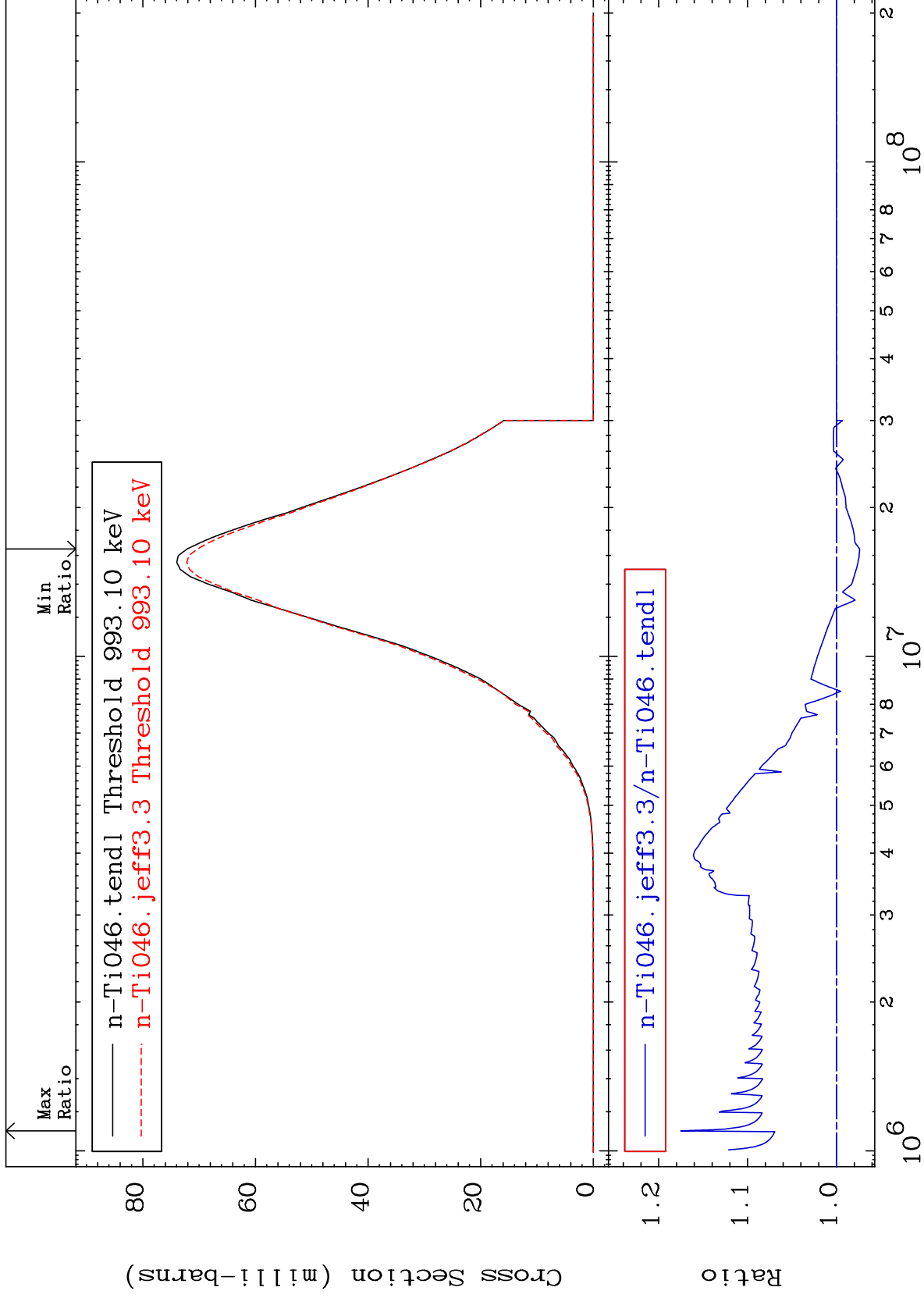
MAT 2225

(n, α)

²²Ti-46

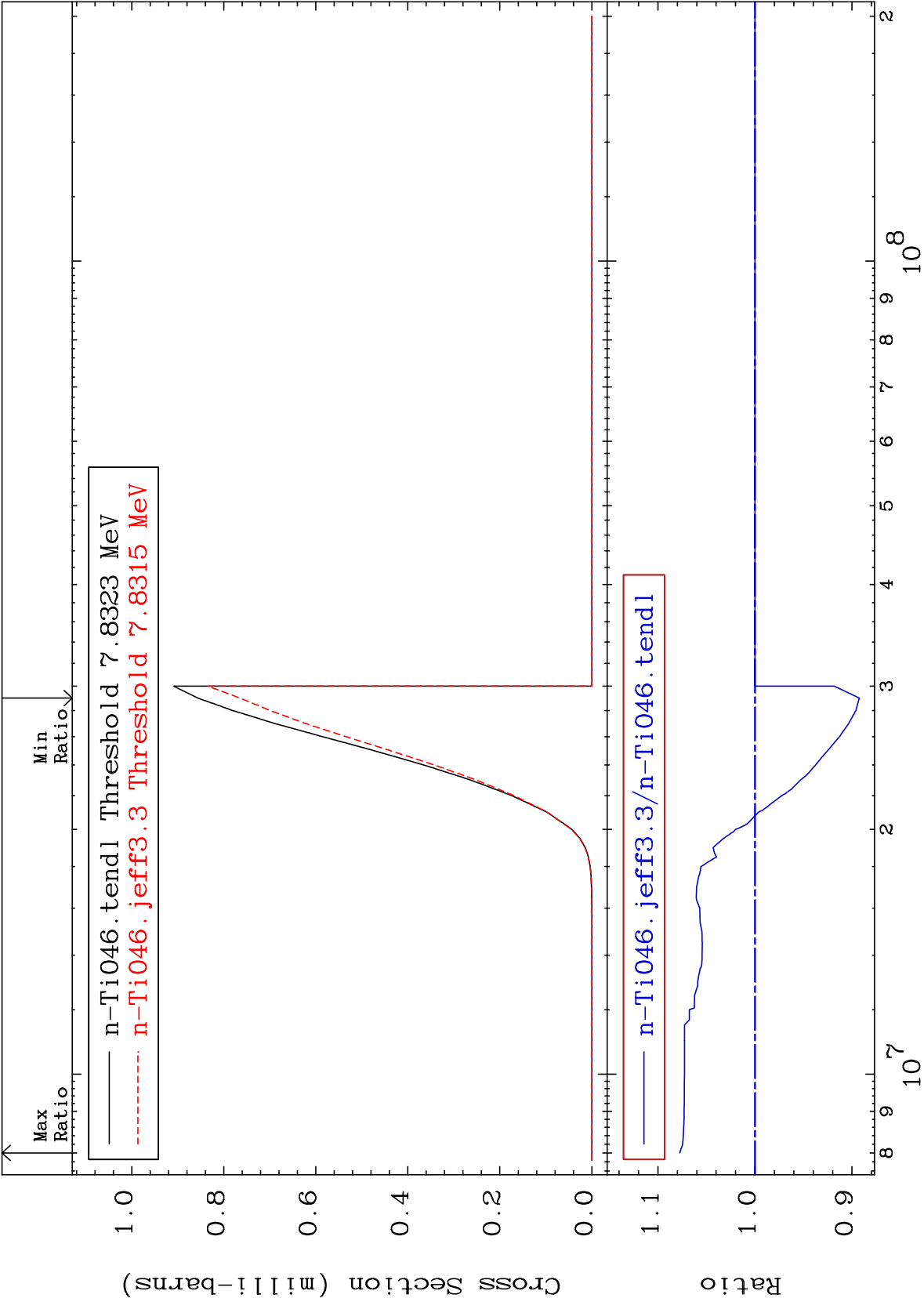
Cross Section

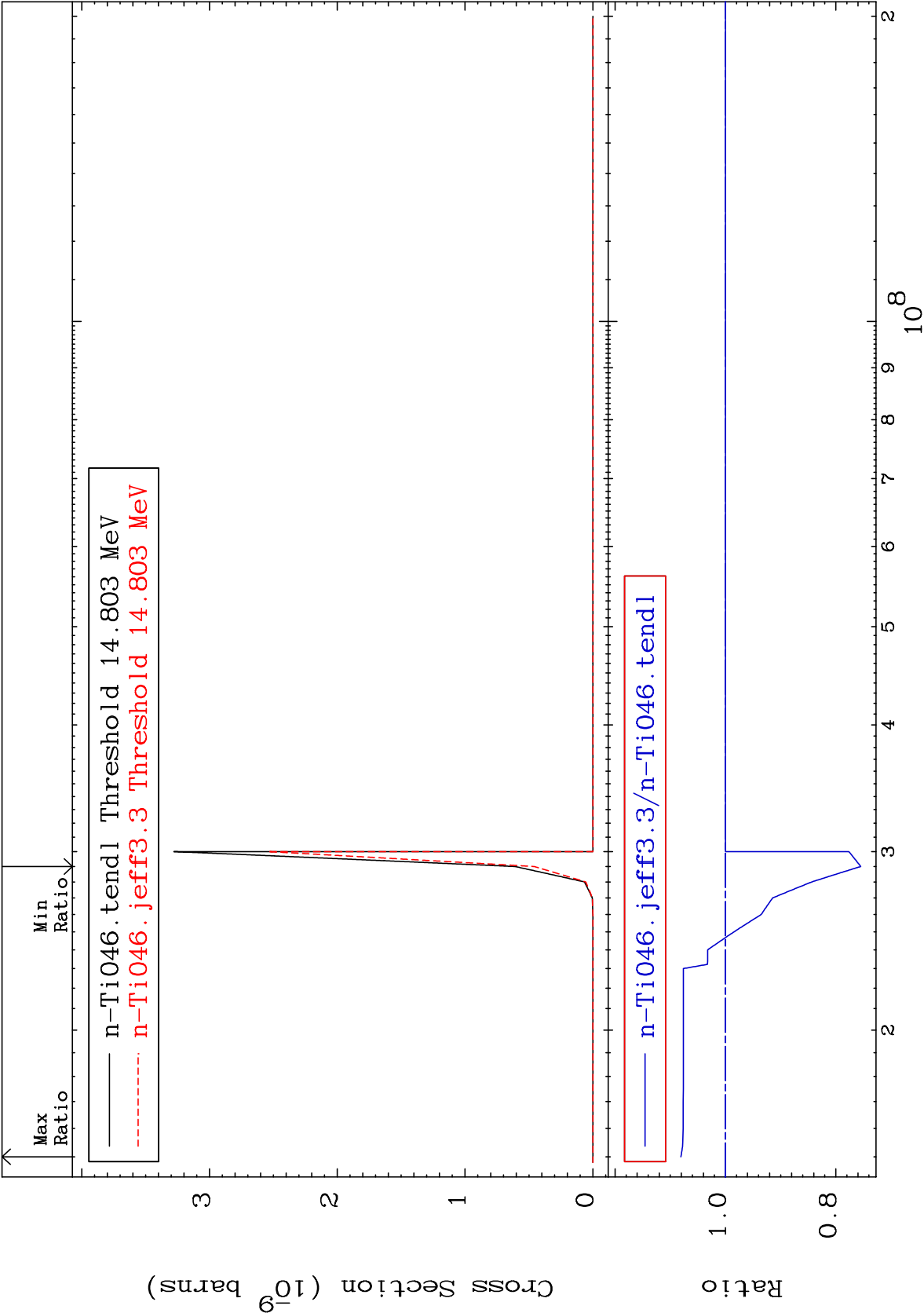
-2.597 To 17.52 %



Incident Energy (eV)

²²Ti-46





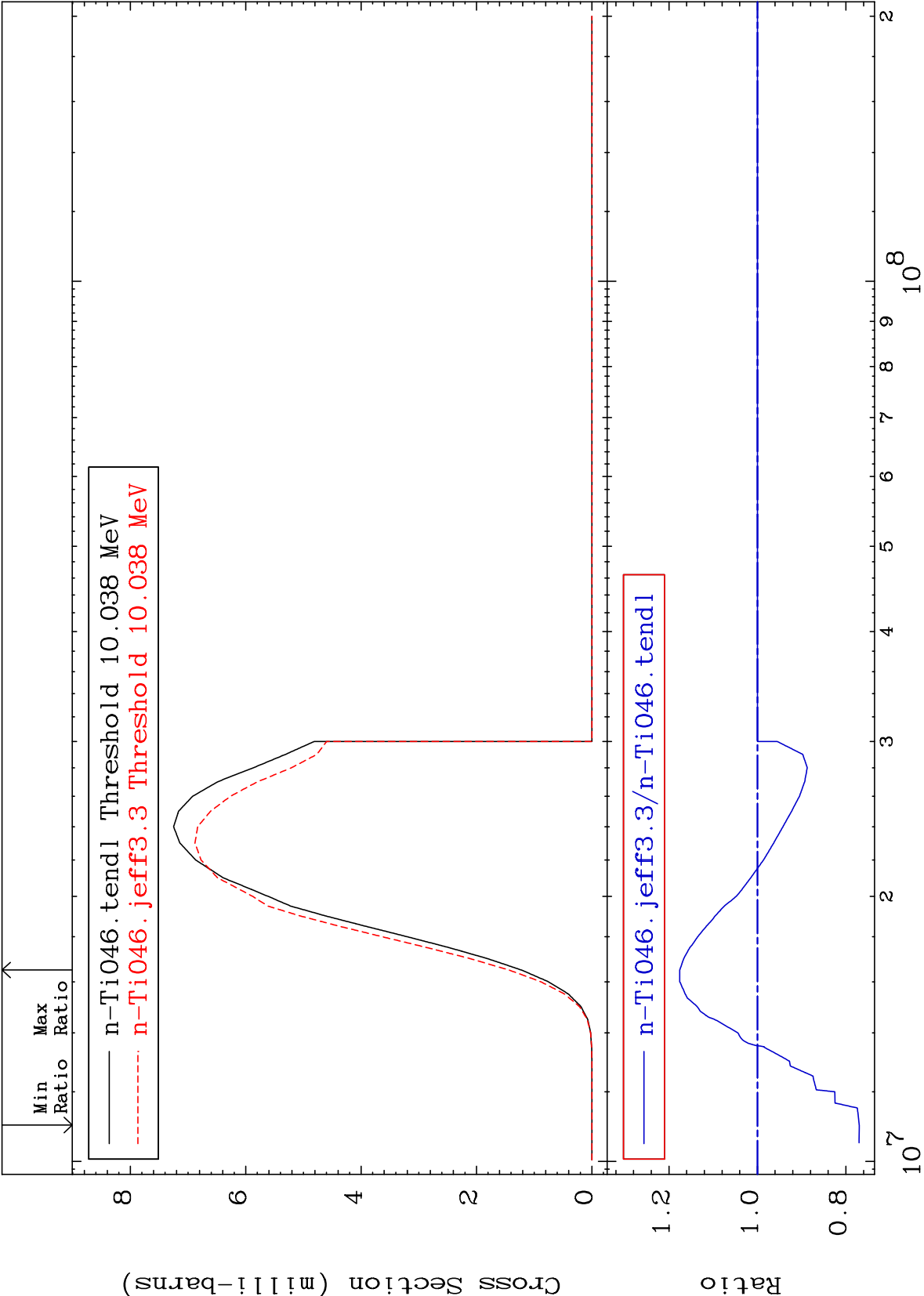
MAT 2225

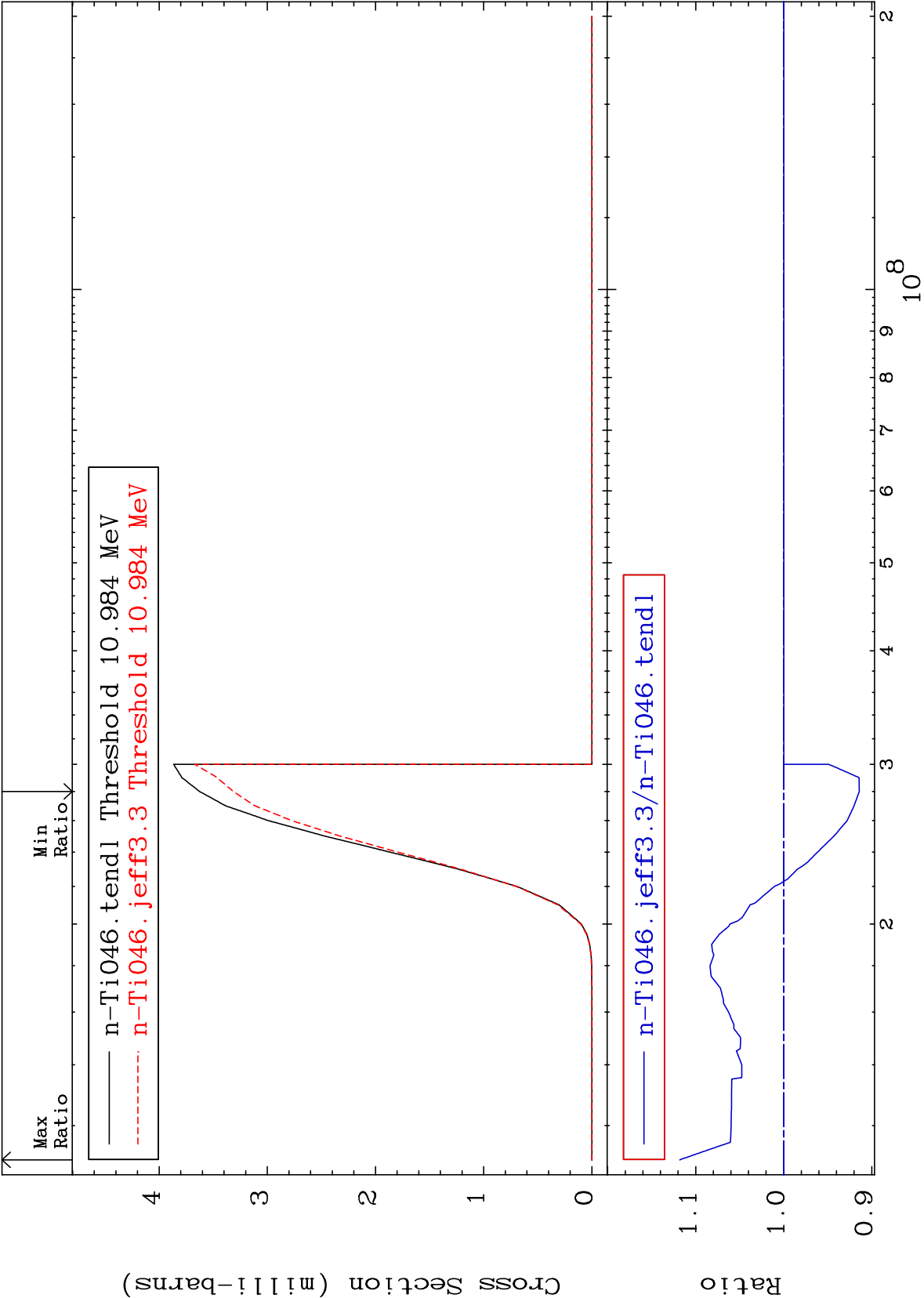
(n,2p)

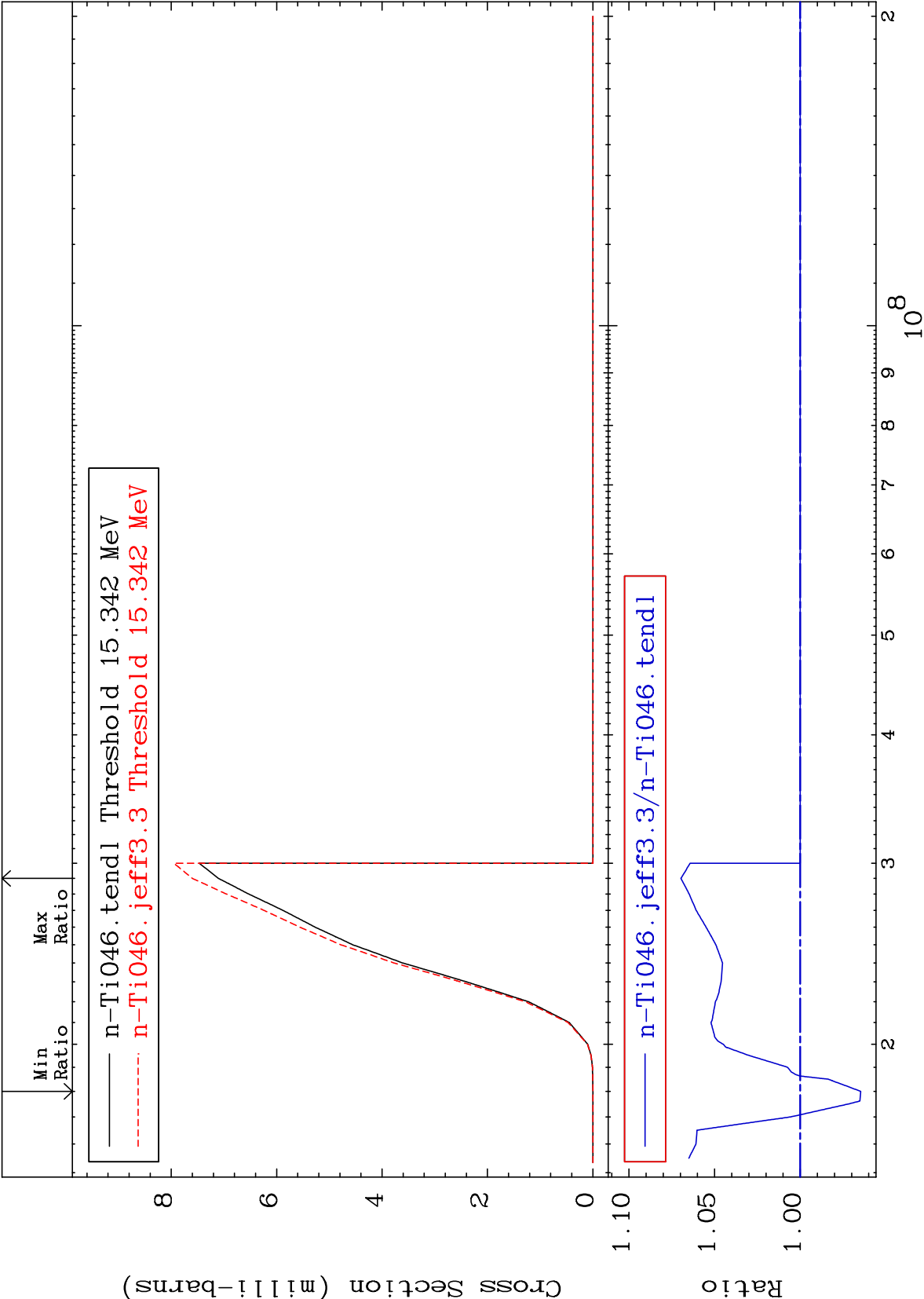
22-Ti-46

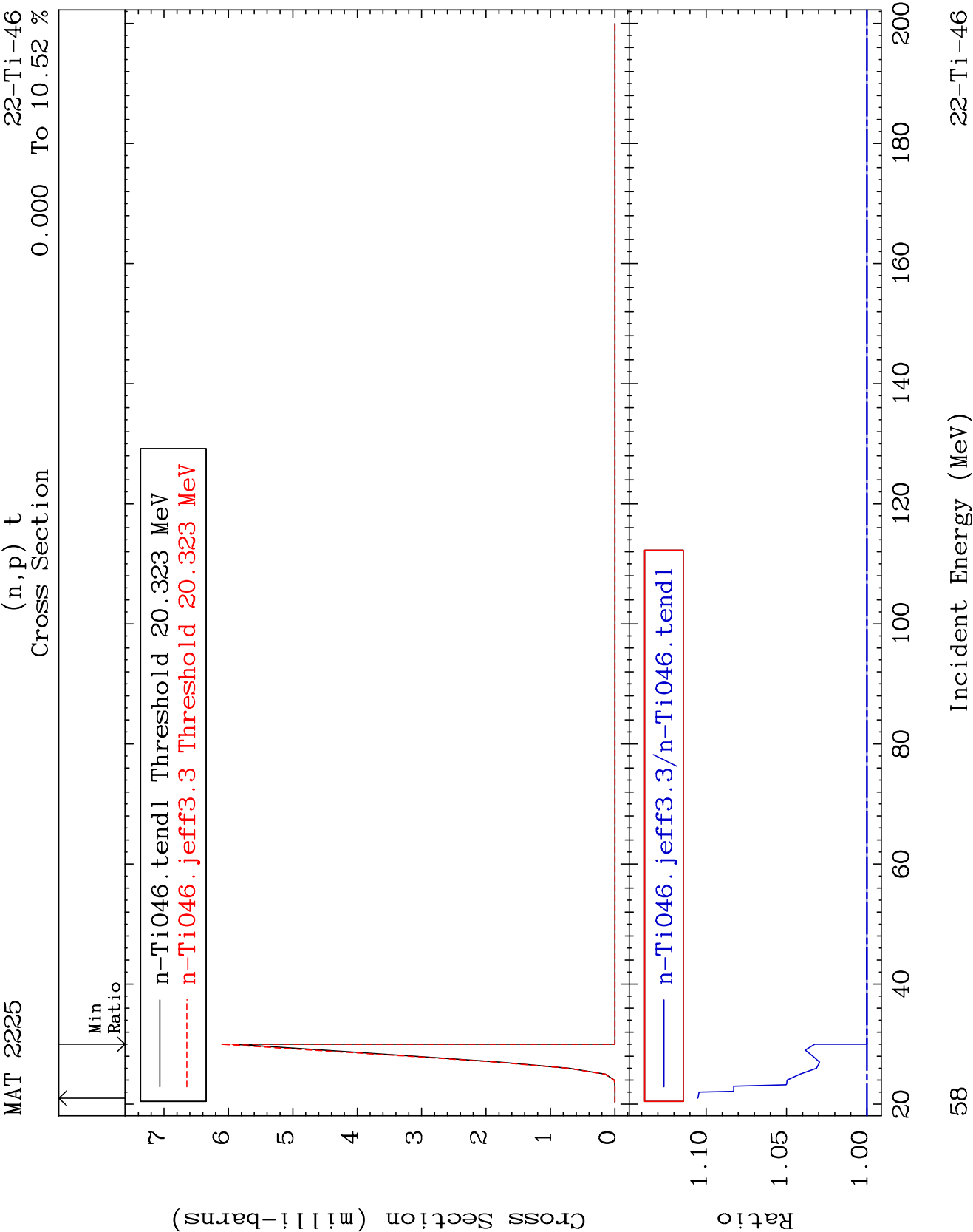
Cross Section

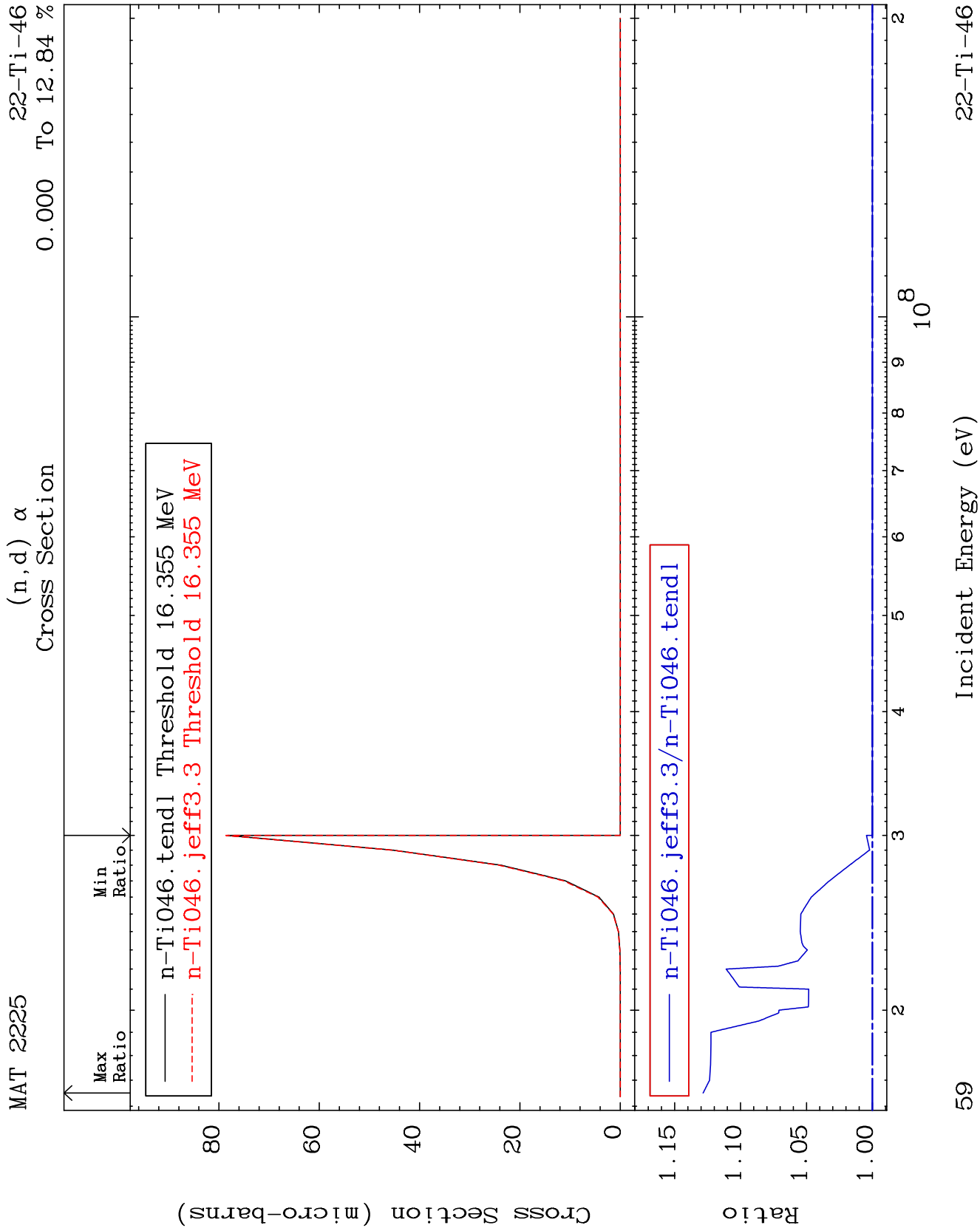
-23.03 To 17.57 %

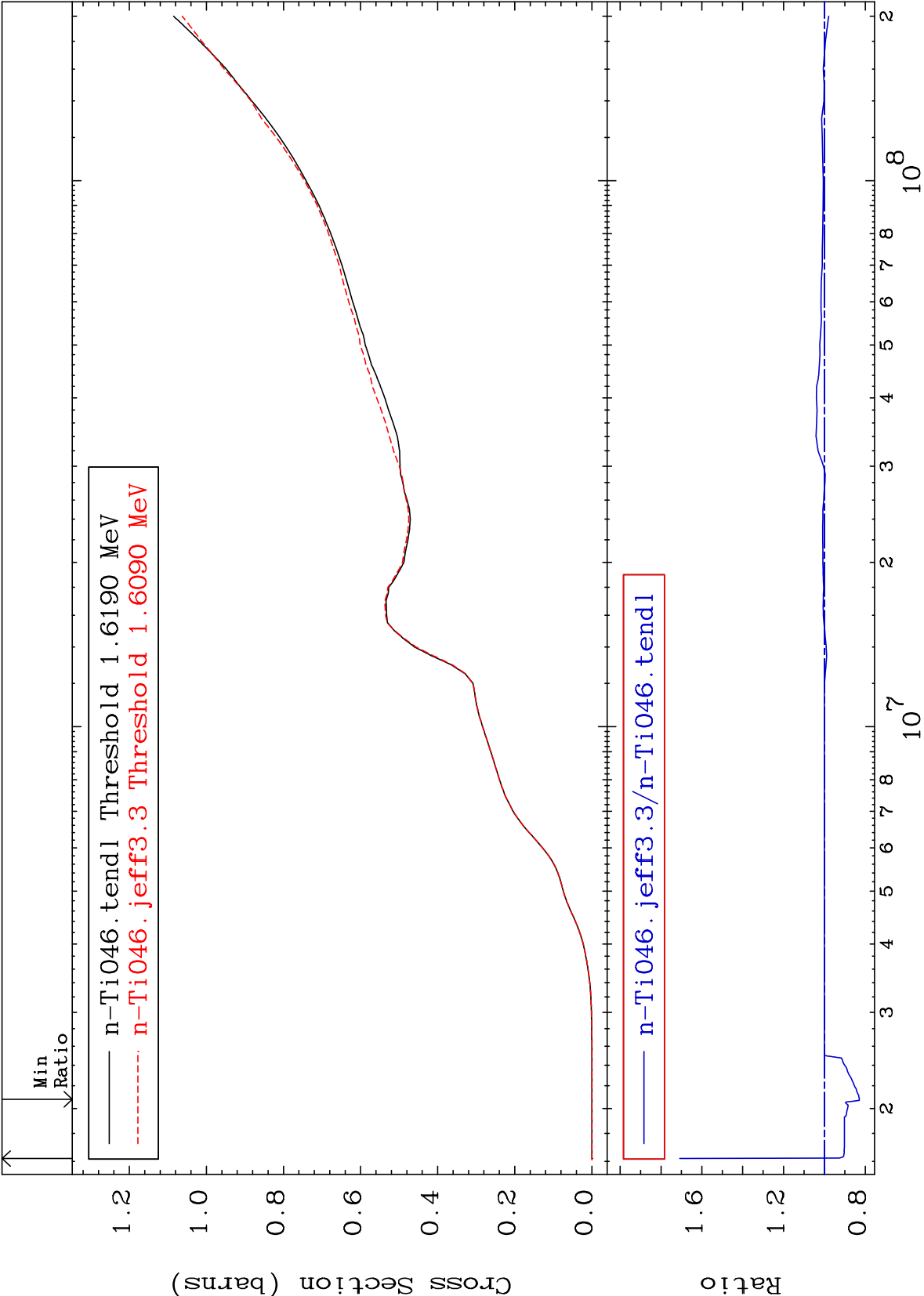


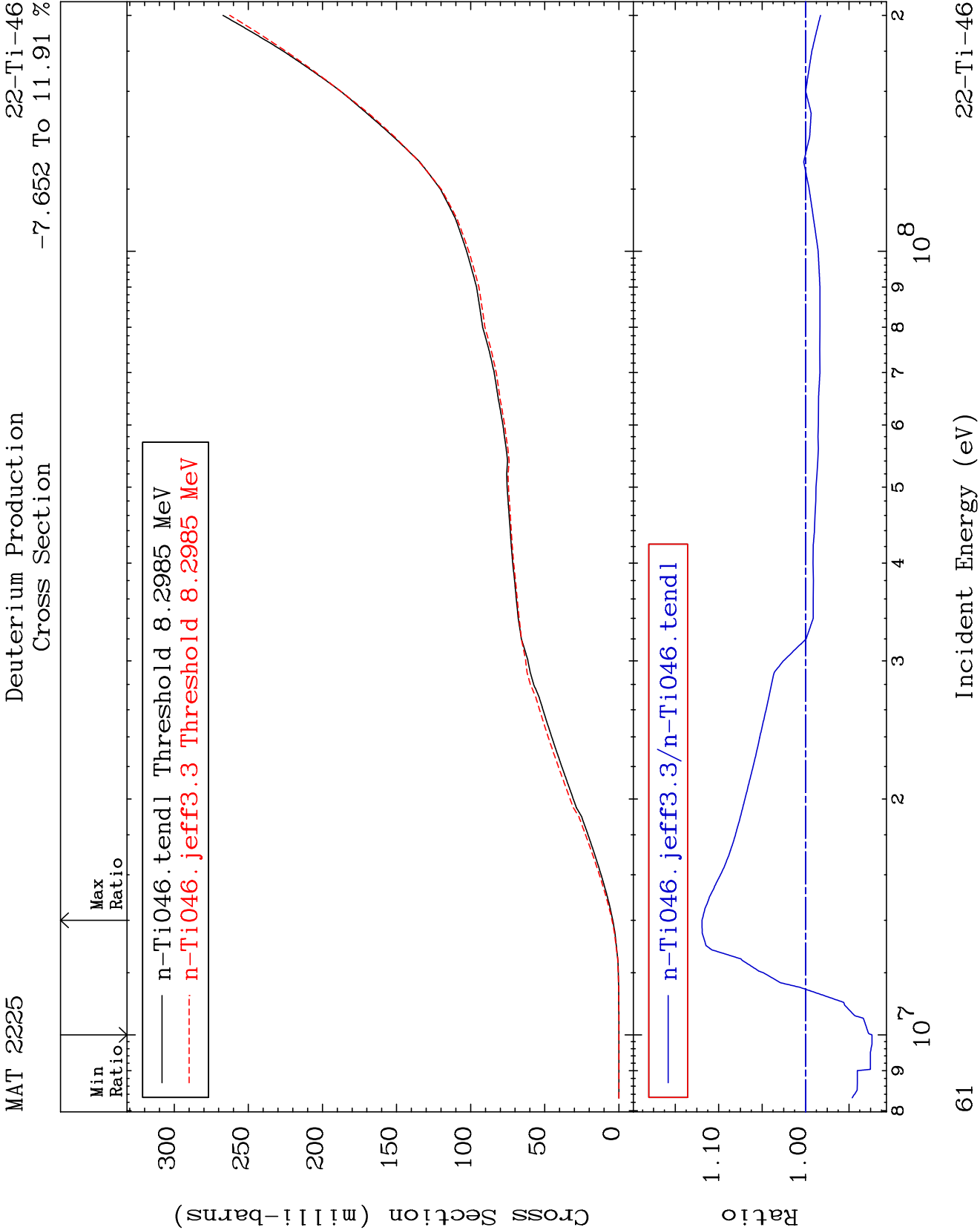


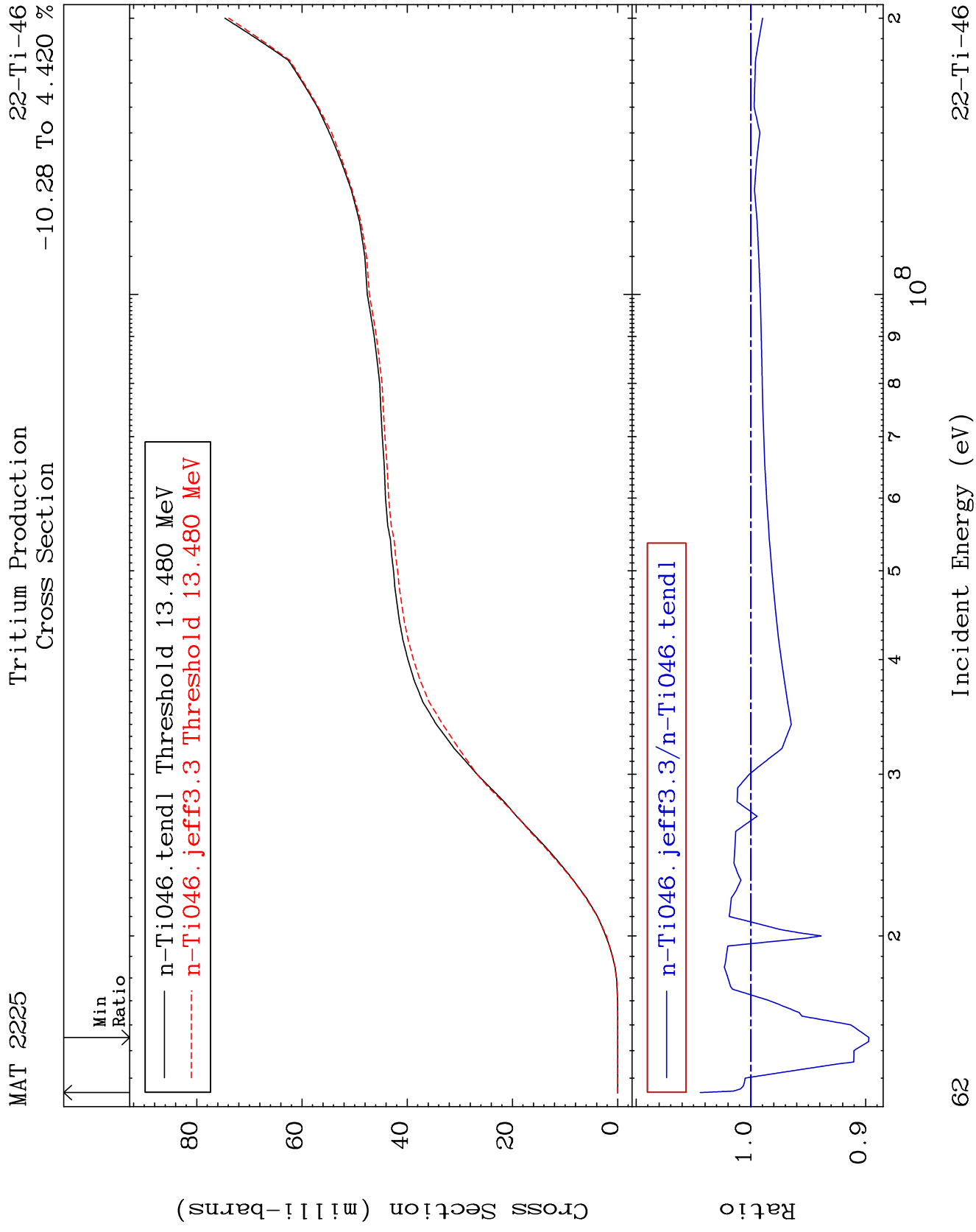








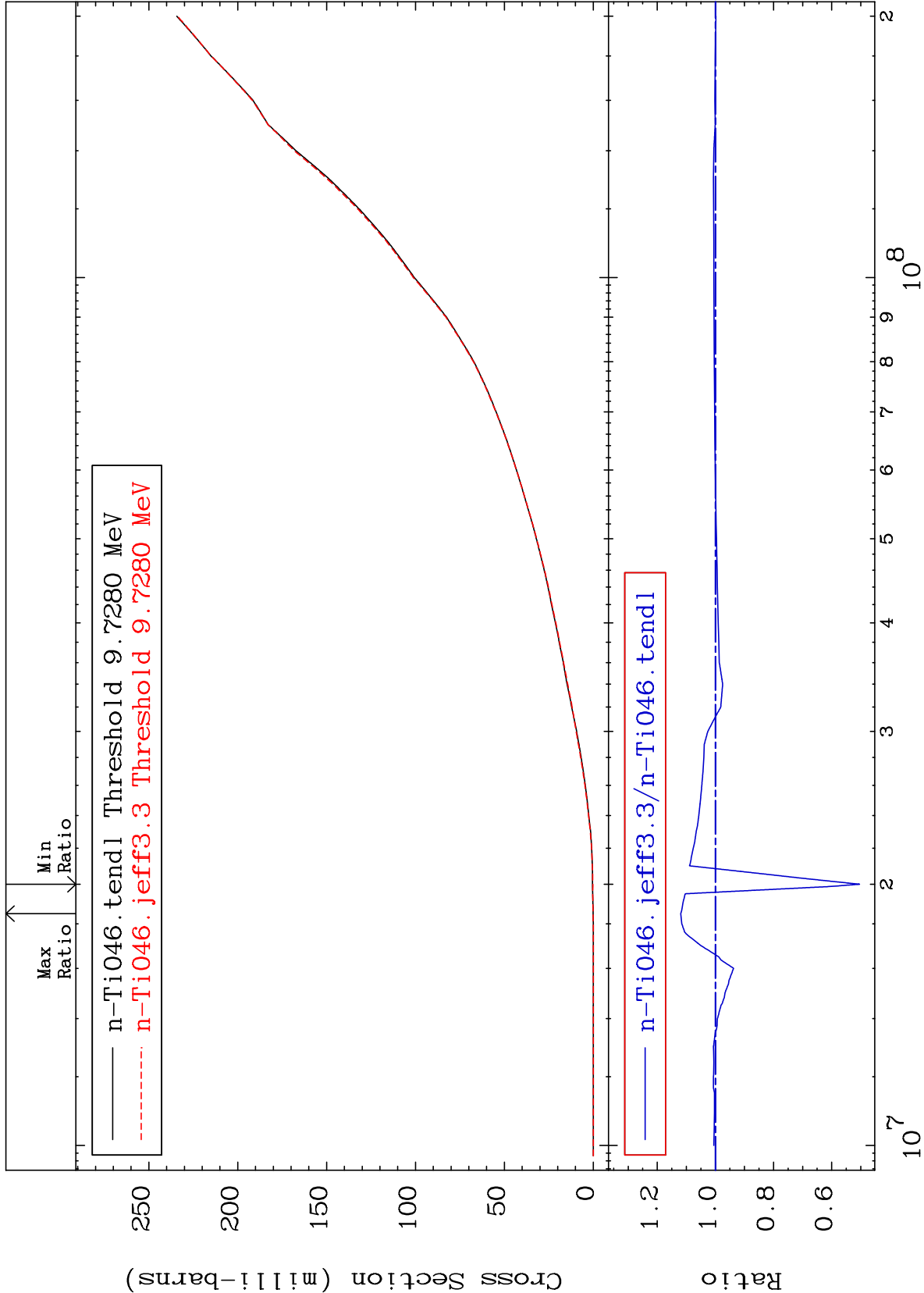




MAT 2225

He-3 Production
Cross Section

22-Ti-46
-49.58 To 11.91 %



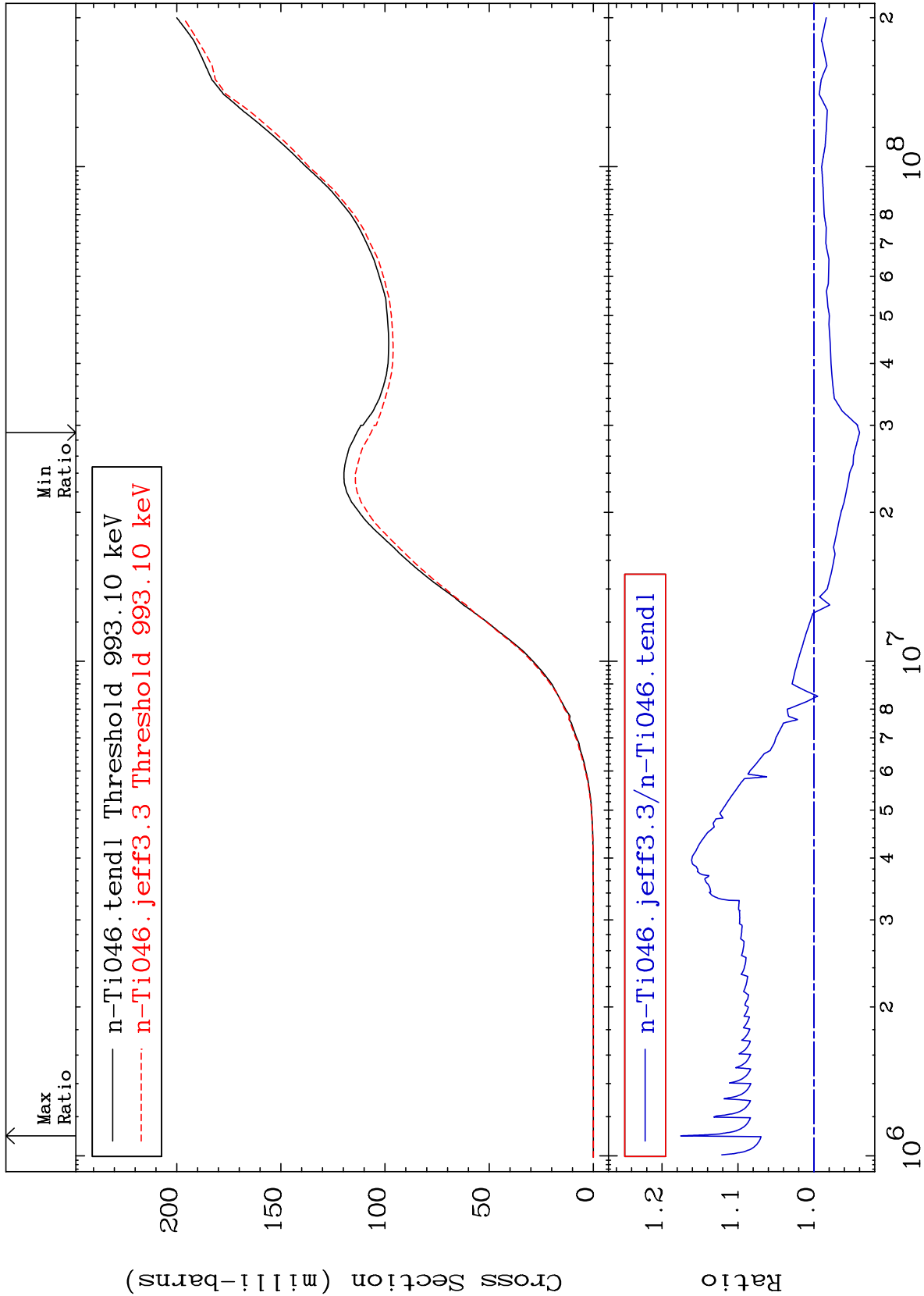
Incident Energy (eV)

22-Ti-46

MAT 2225

He-4 Production
Cross Section

22-Ti-46
-5.995 To 17.52 %

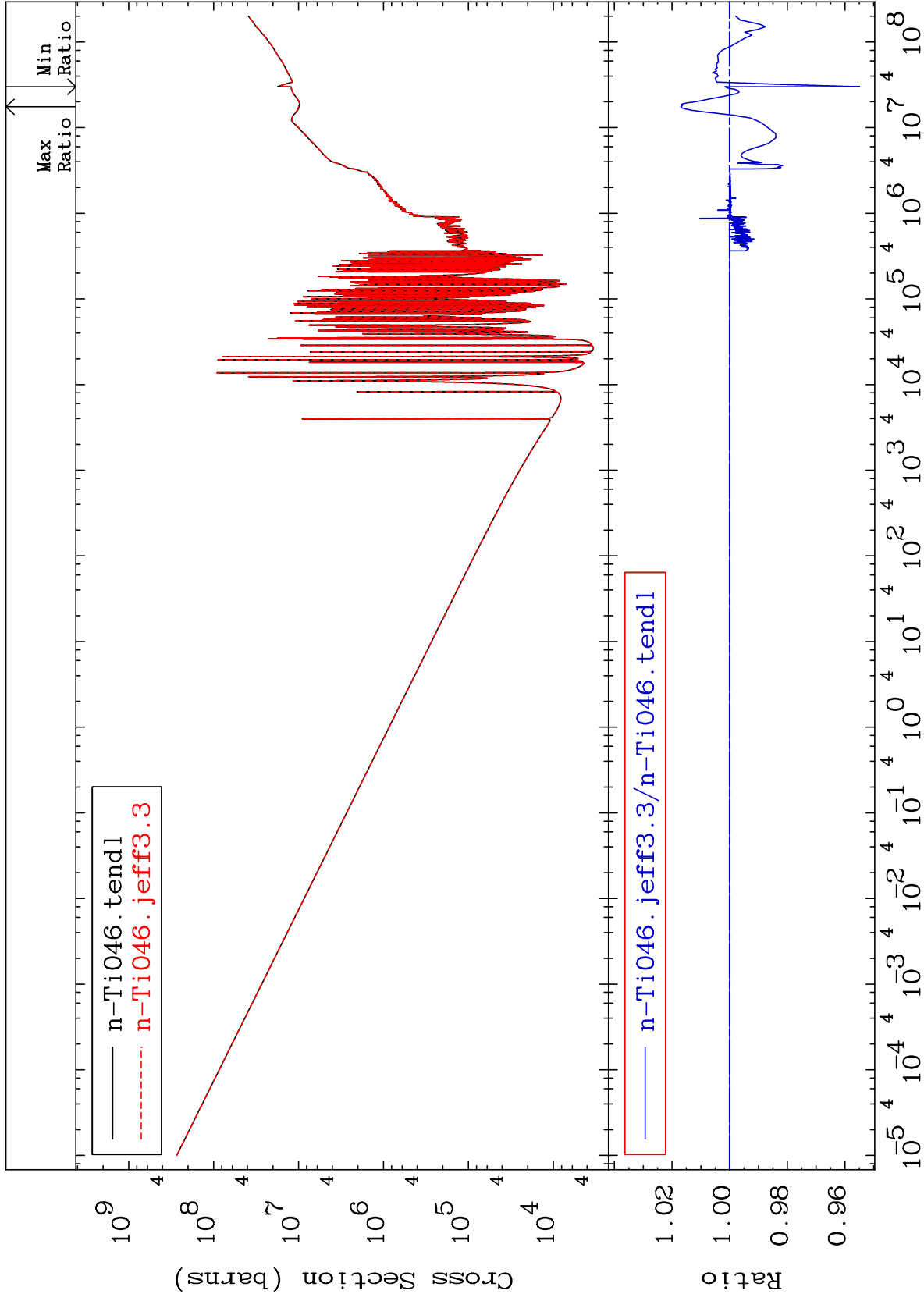


Incident Energy (eV)

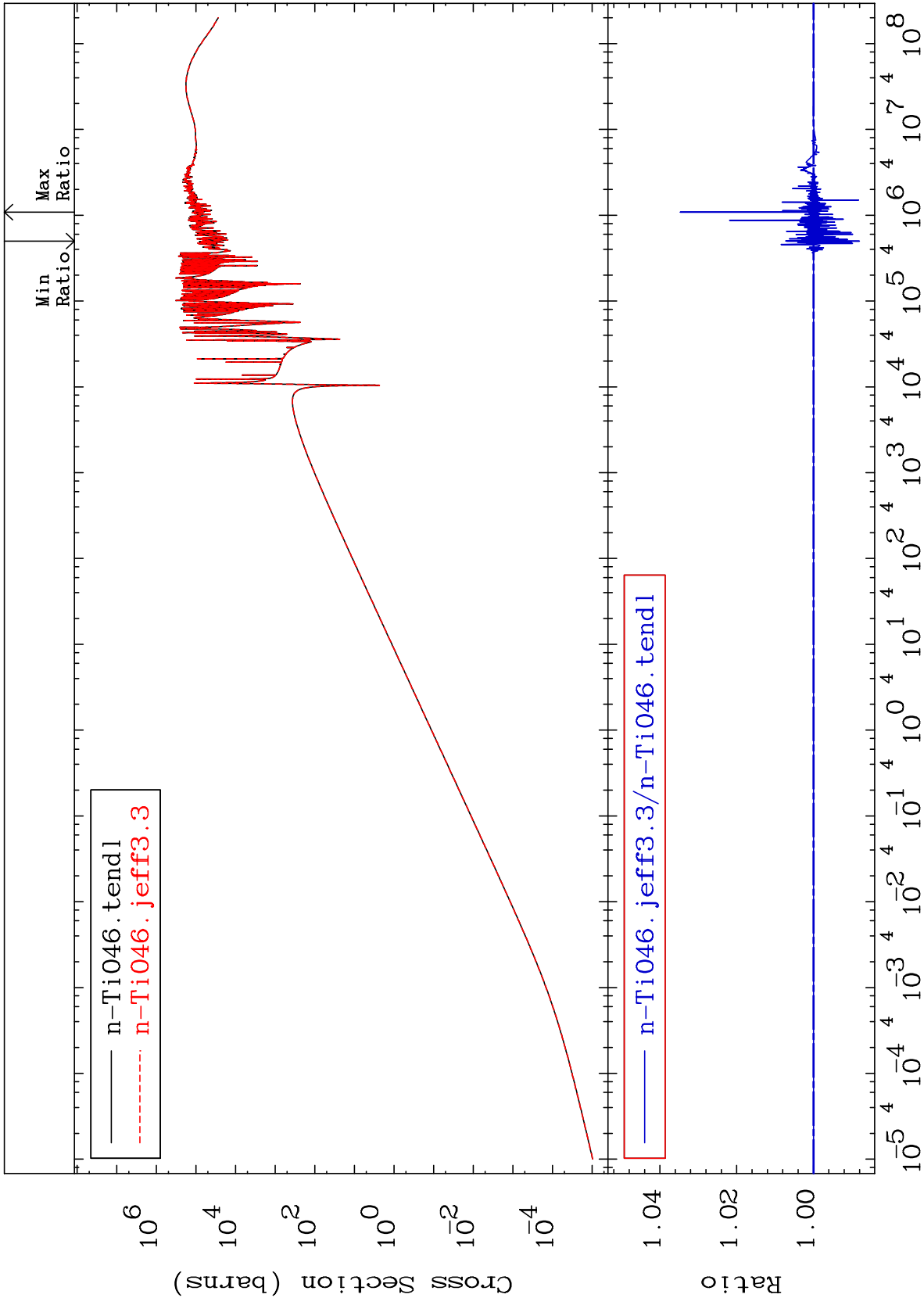
22-Ti-46

Cross Section

-4.507 To 1.697 %

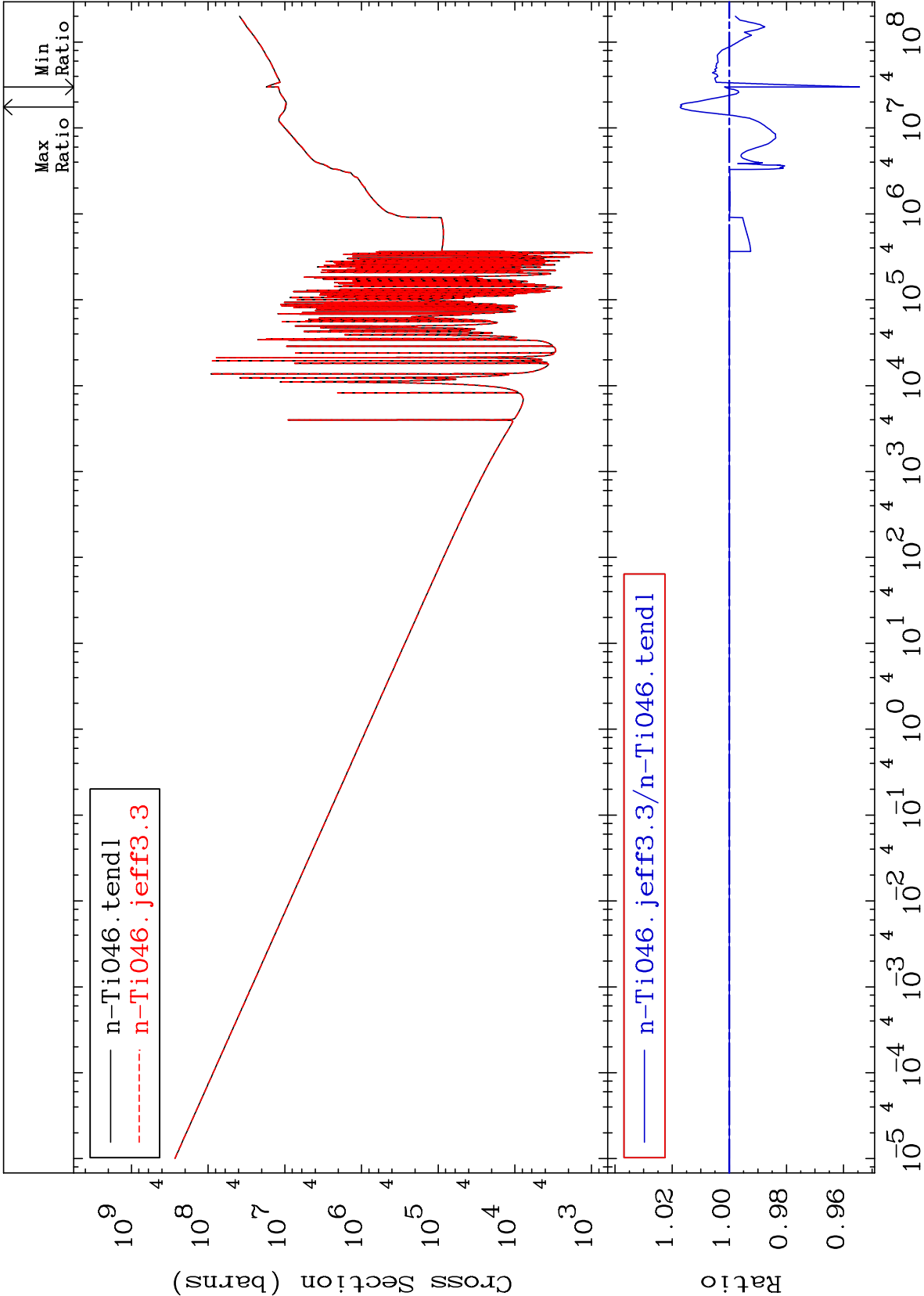


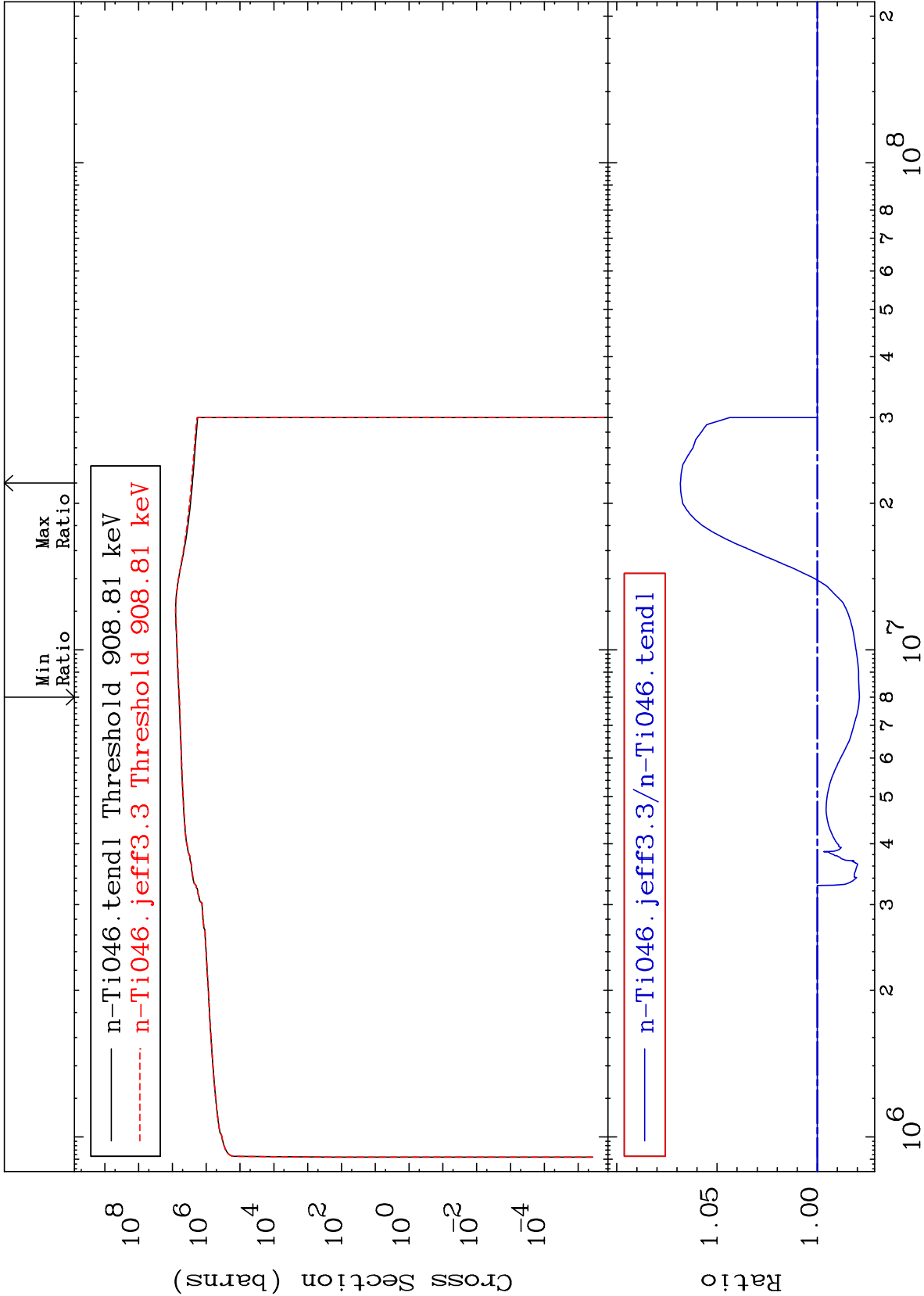
Incident Energy (eV)

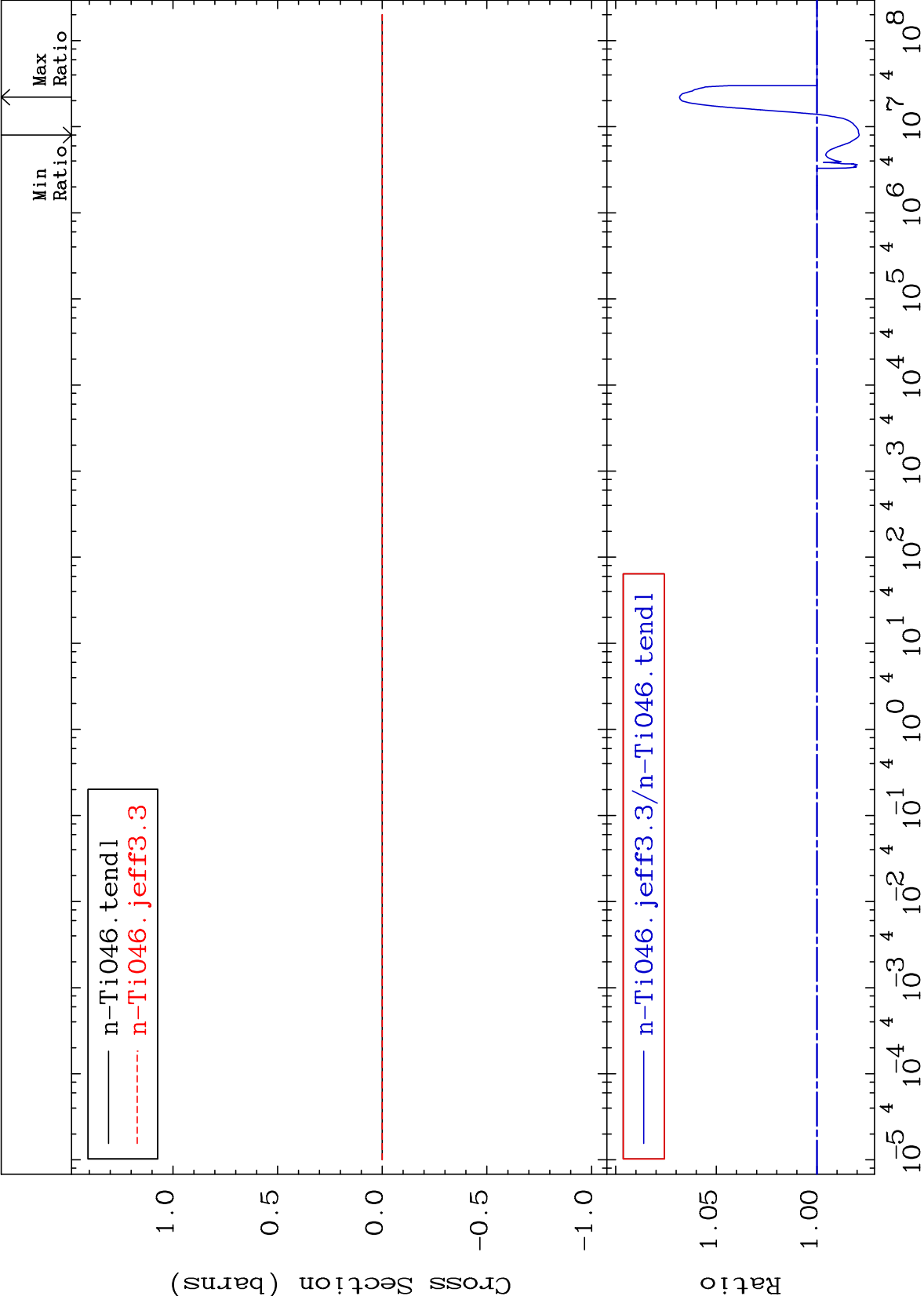


Cross Section

-4.553 To 1.722 %



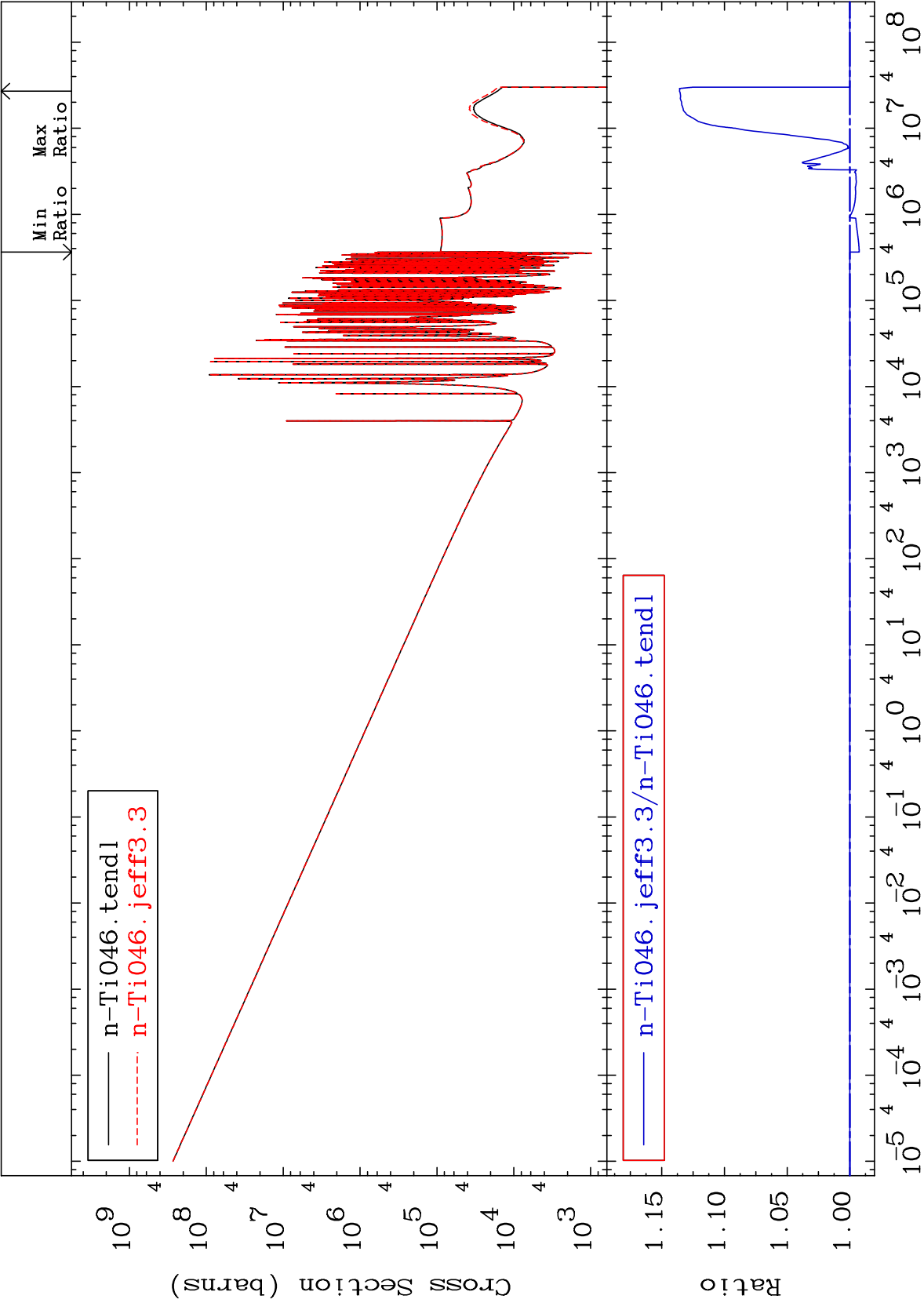




MAT 2225

Kerma capture (mt102)
Cross Section

22-Ti-46
-0.748 To 13.58 %



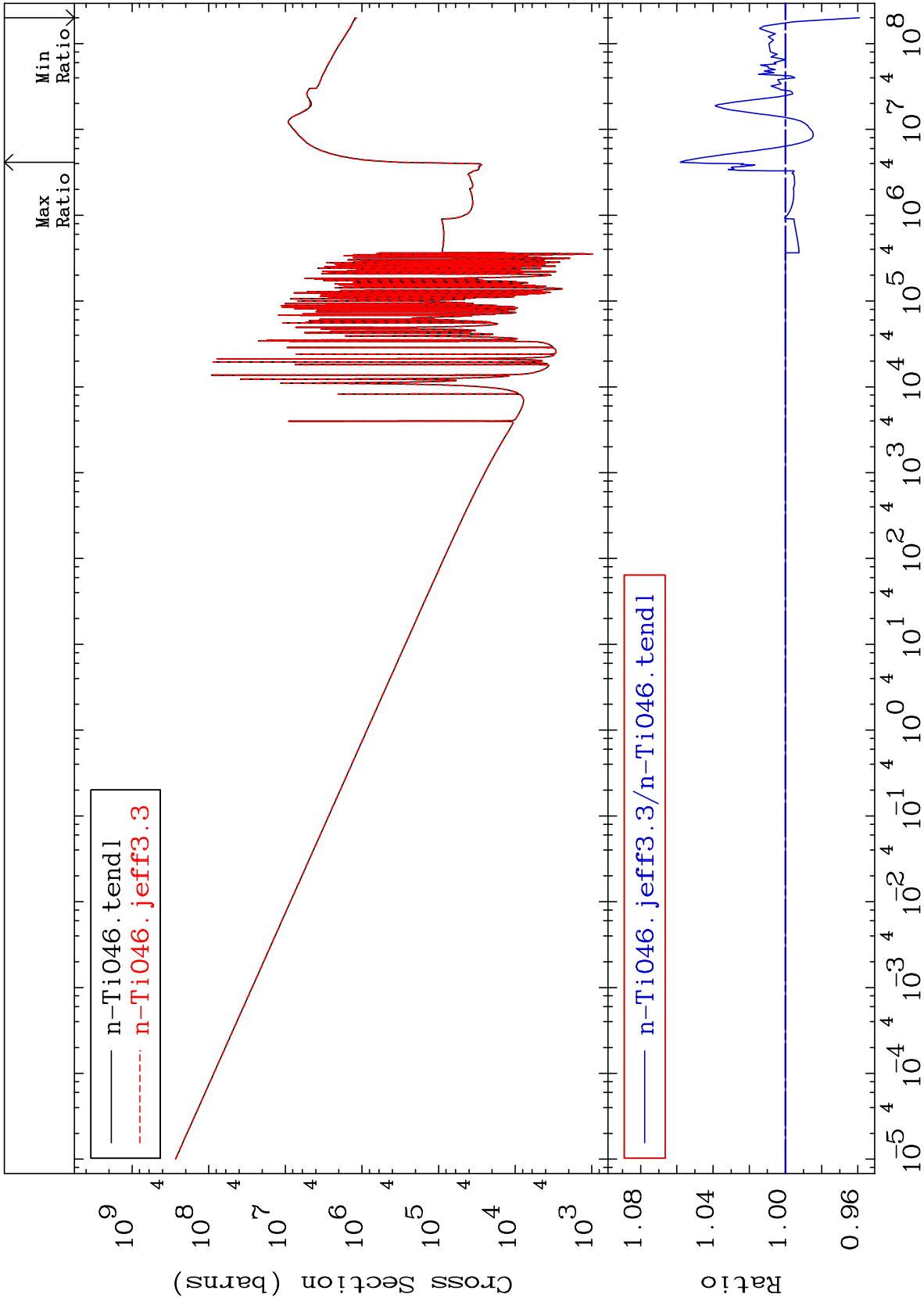
70

Incident Energy (eV)

22-Ti-46

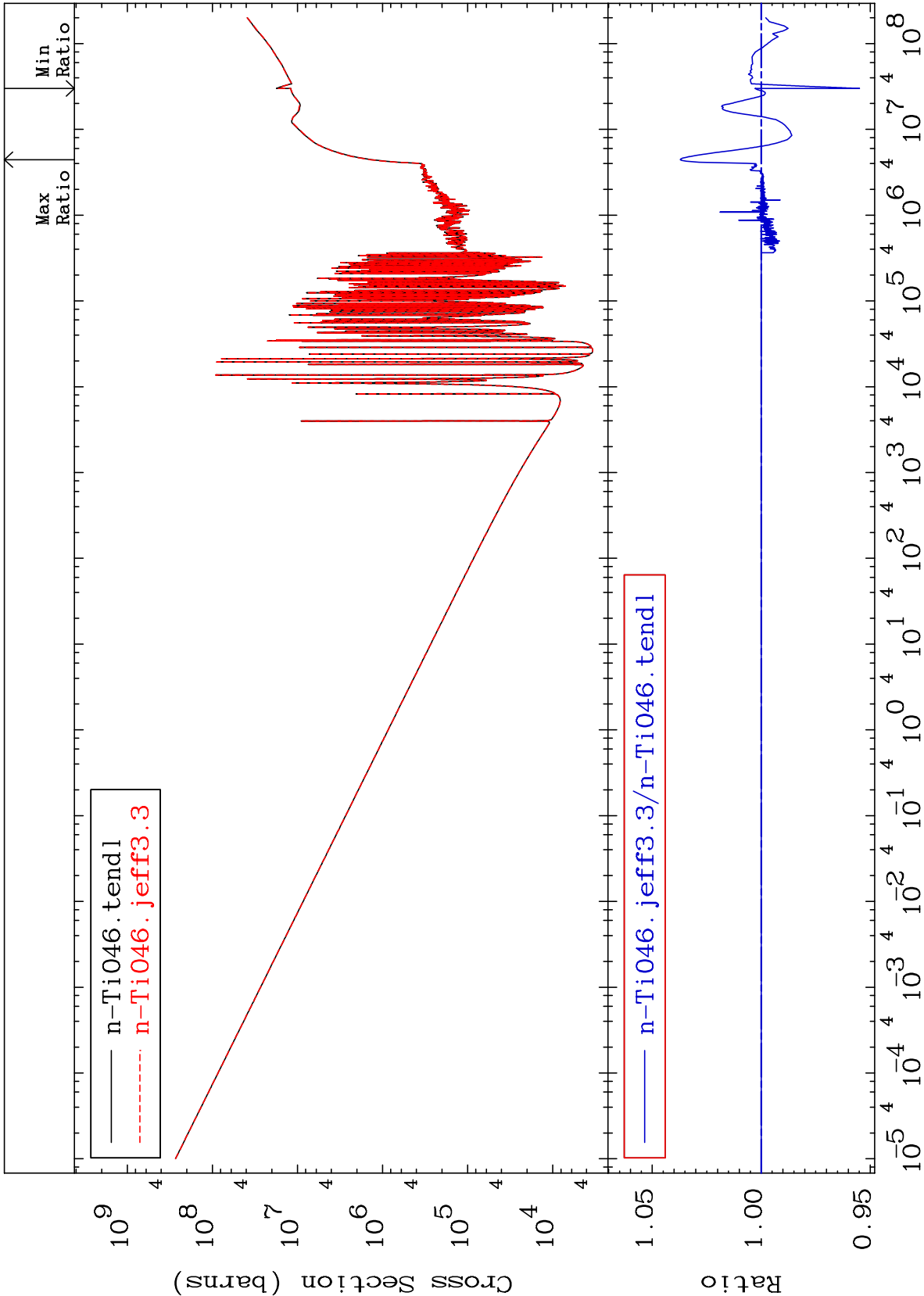
Cross Section

-4.088 To 5.806 %



Cross Section

-4.507 To 3.710 %



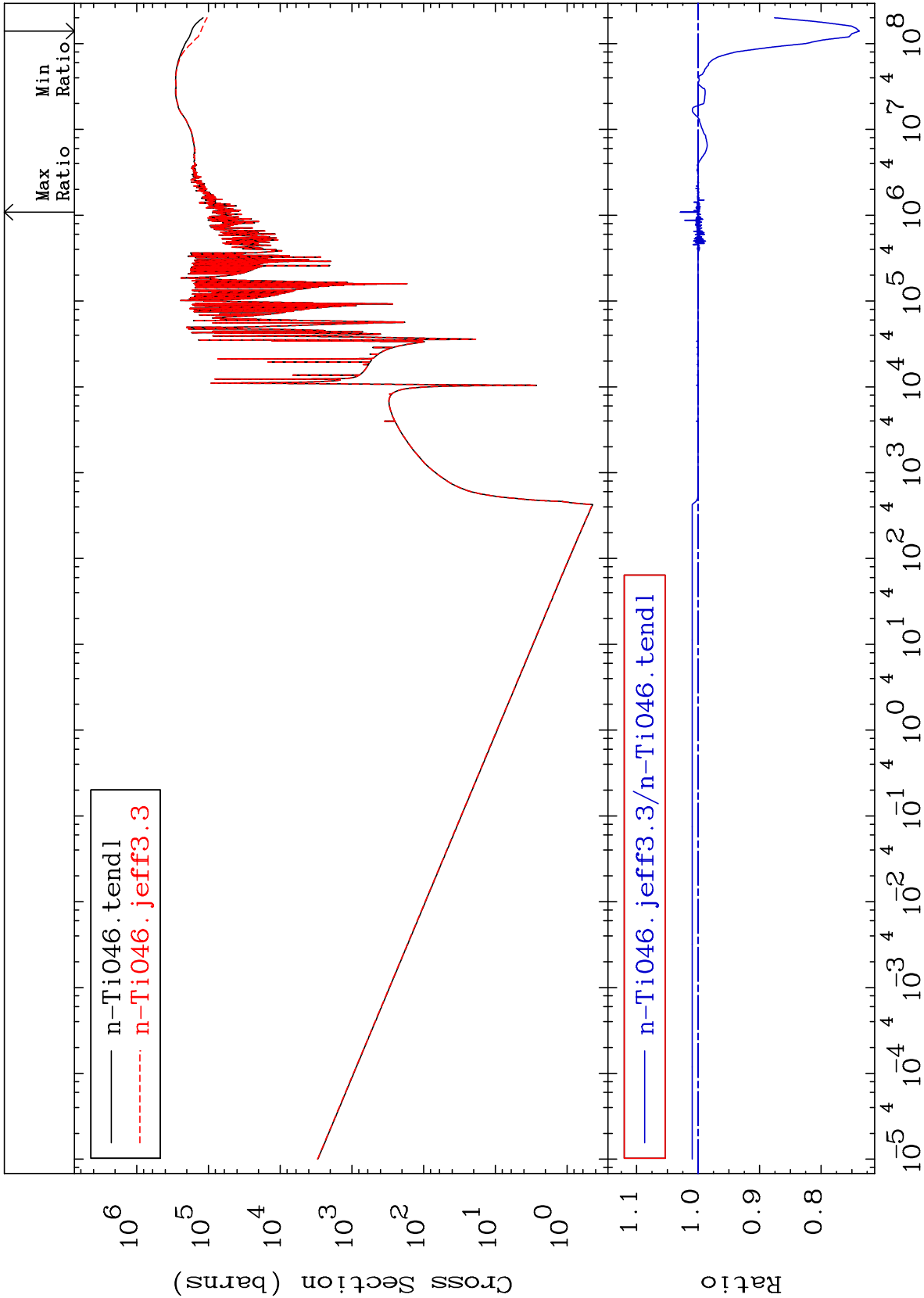
MAT 2225

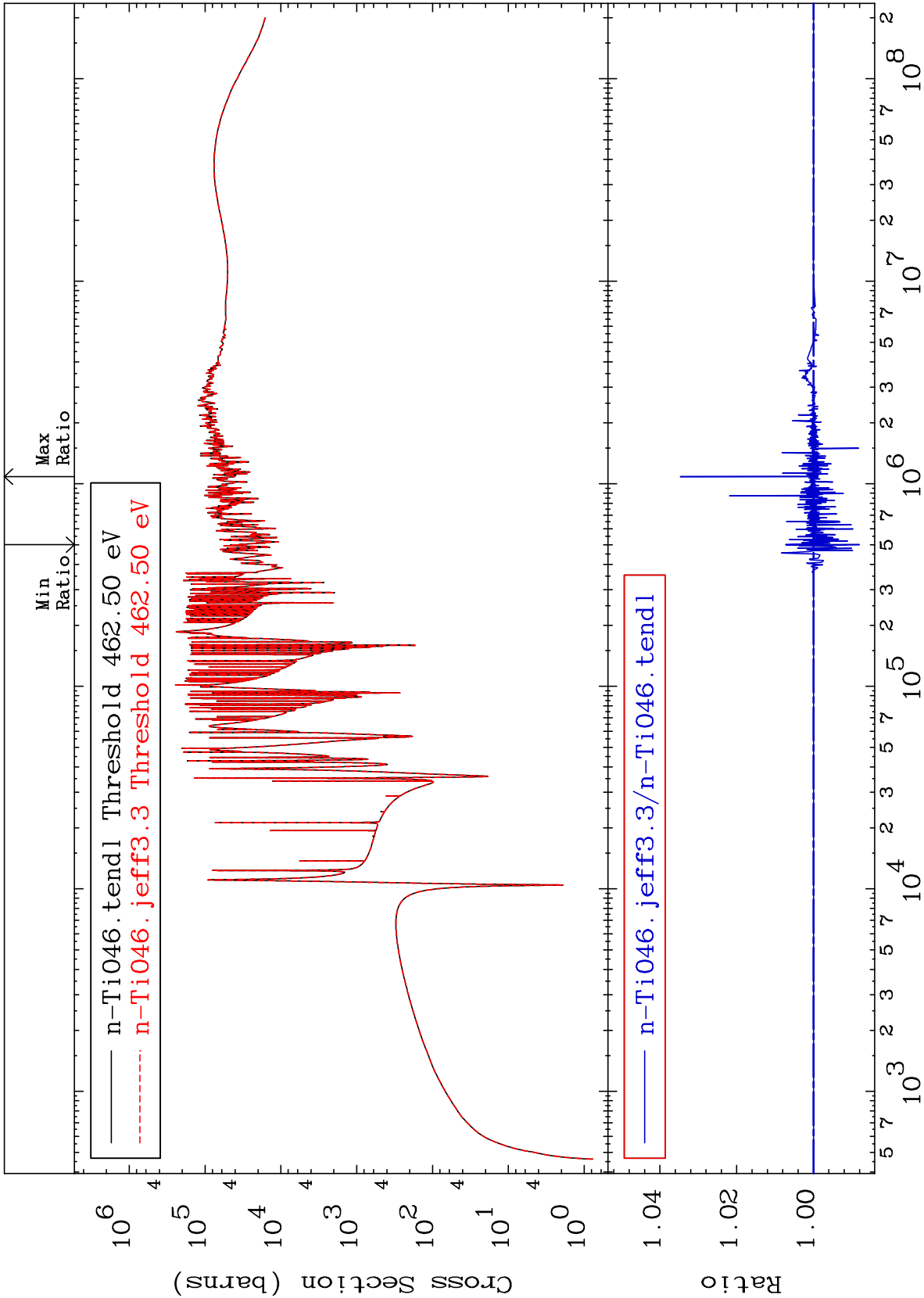
Dpa total (eV-barns)

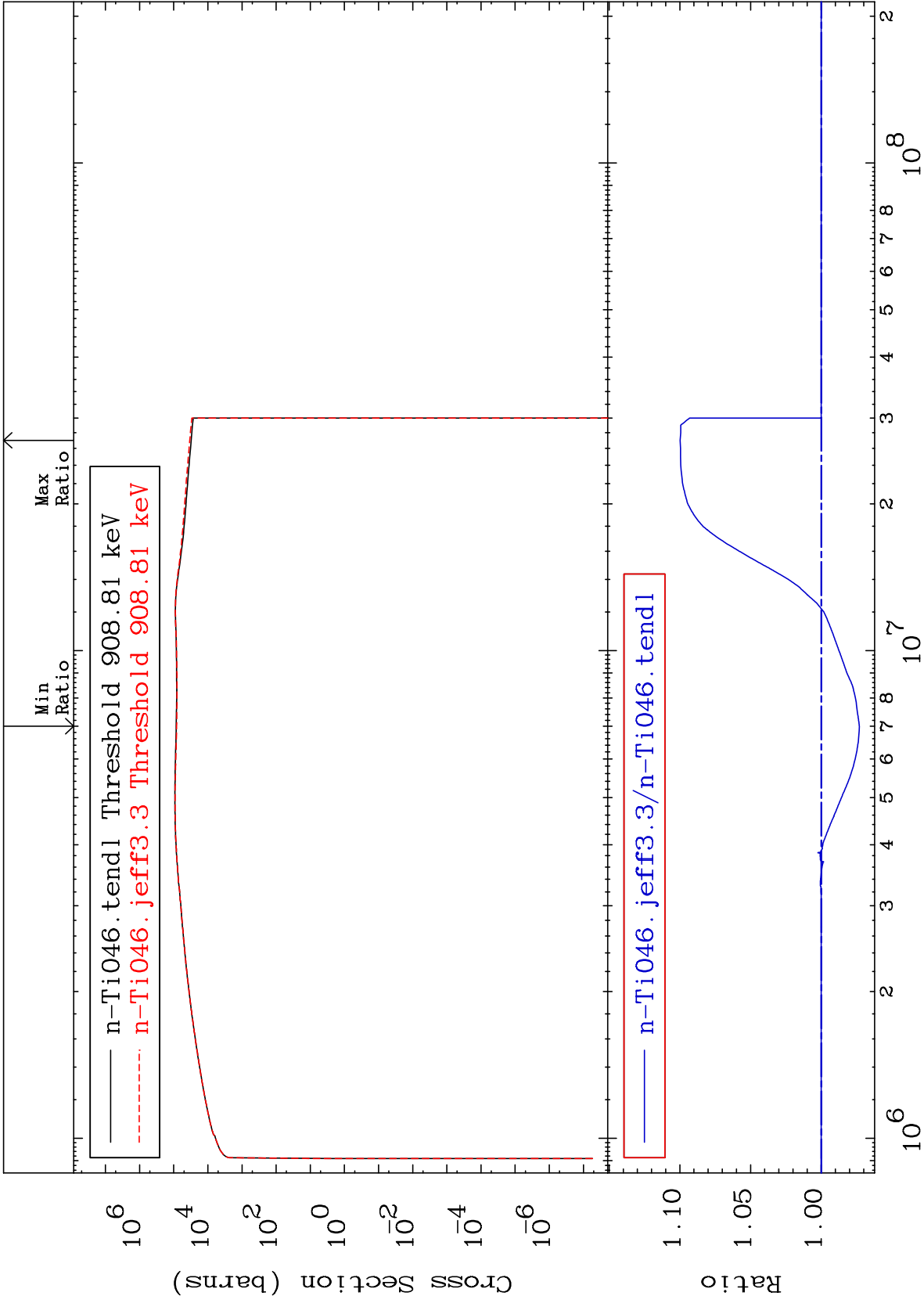
22-Ti-46

Cross Section

-26.24 To 2.886 %

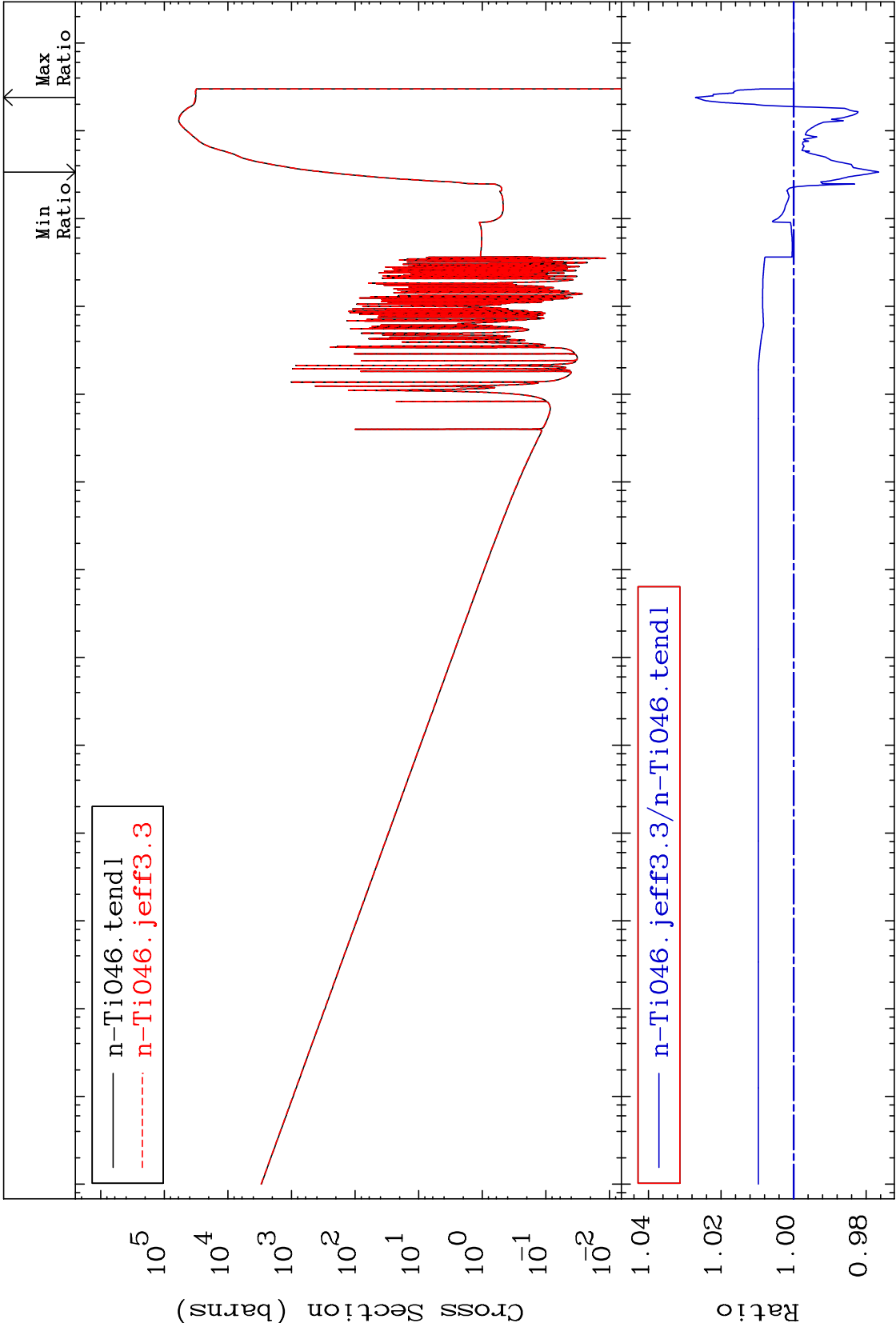


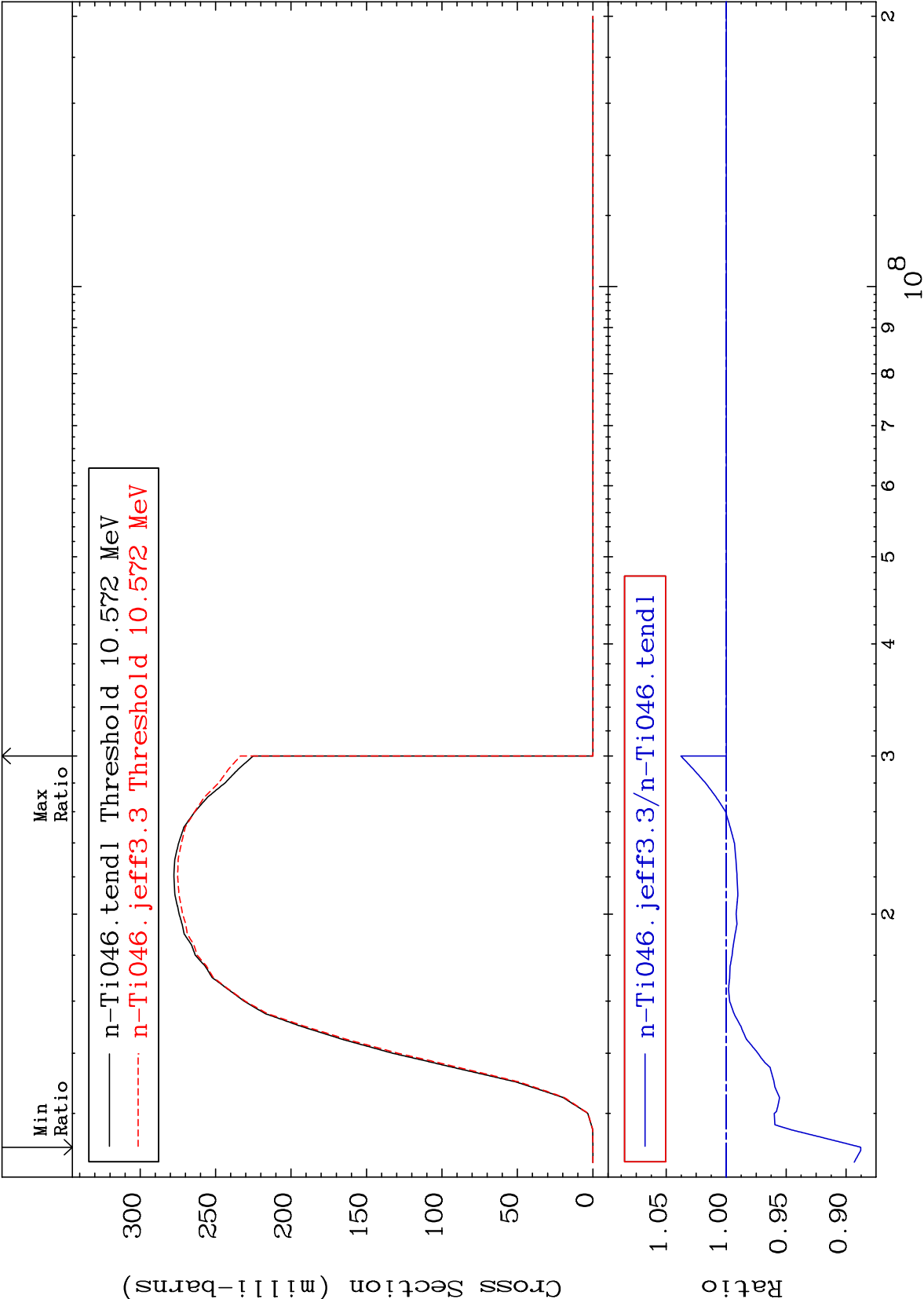




Cross Section

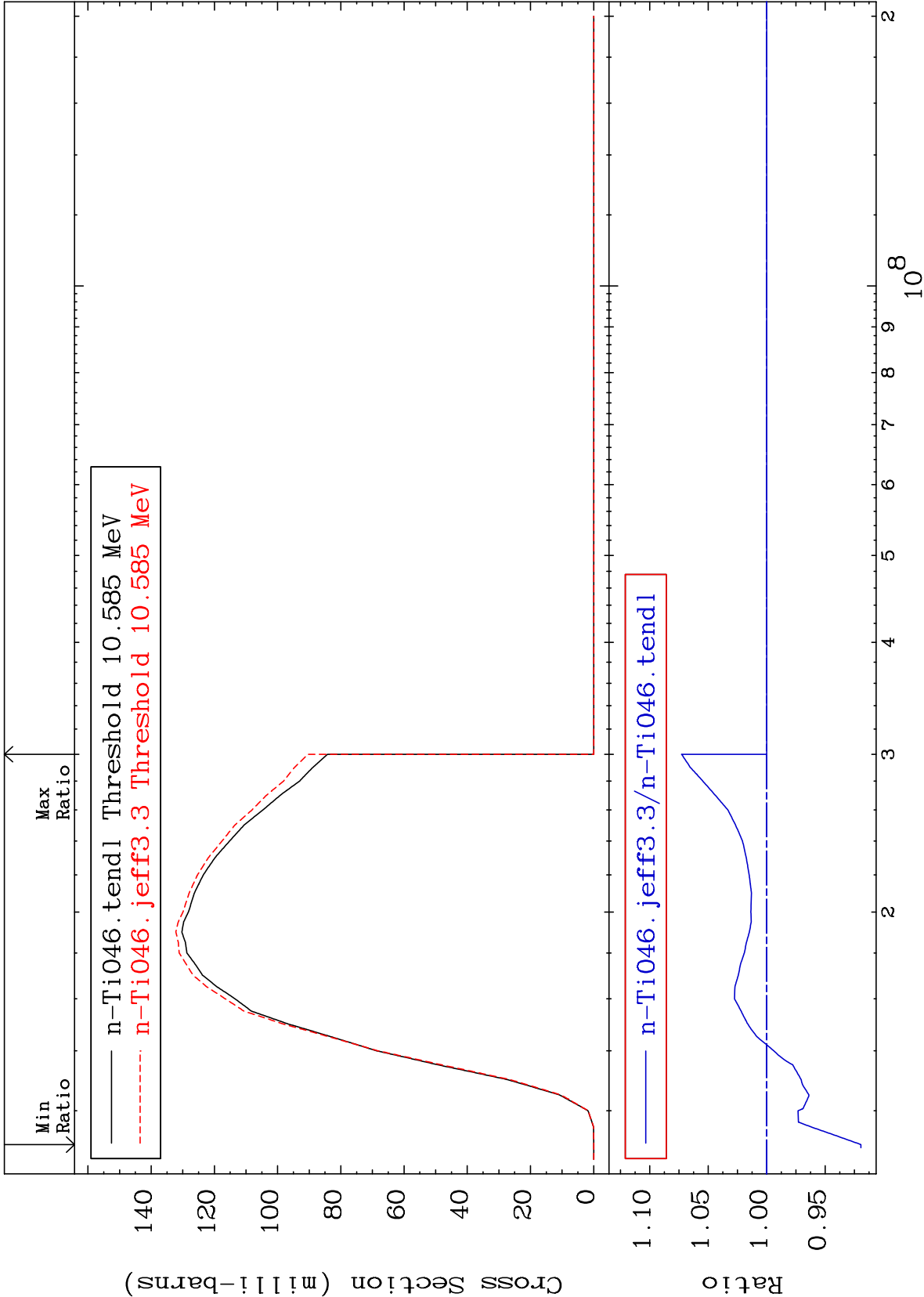
-2.346 To 2.707 %

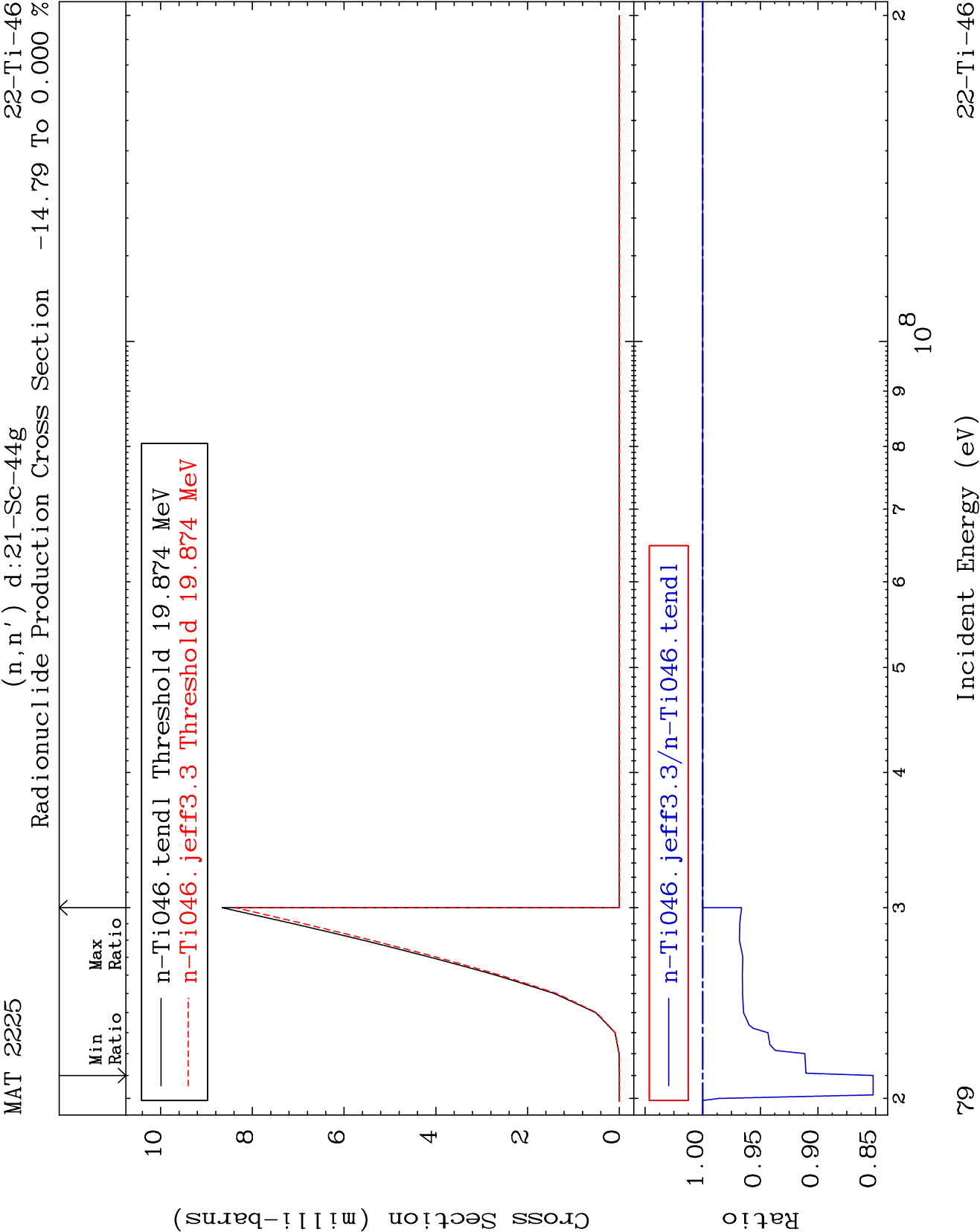




Radionuclide Production Cross Section

-8.060 To 7.276 %





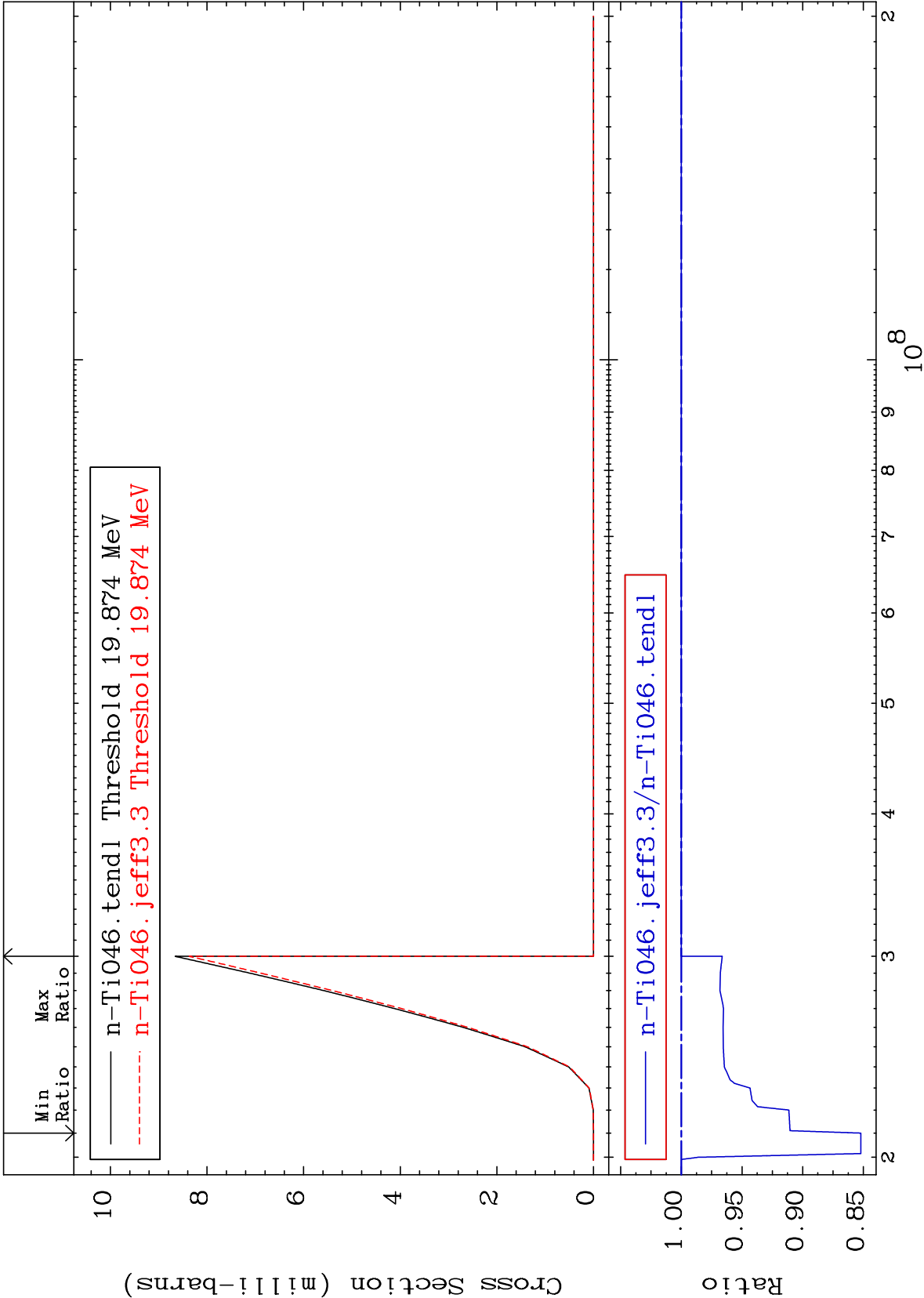
MAT 2225

(n,n') d:21-Sc-44g

22-Ti-46

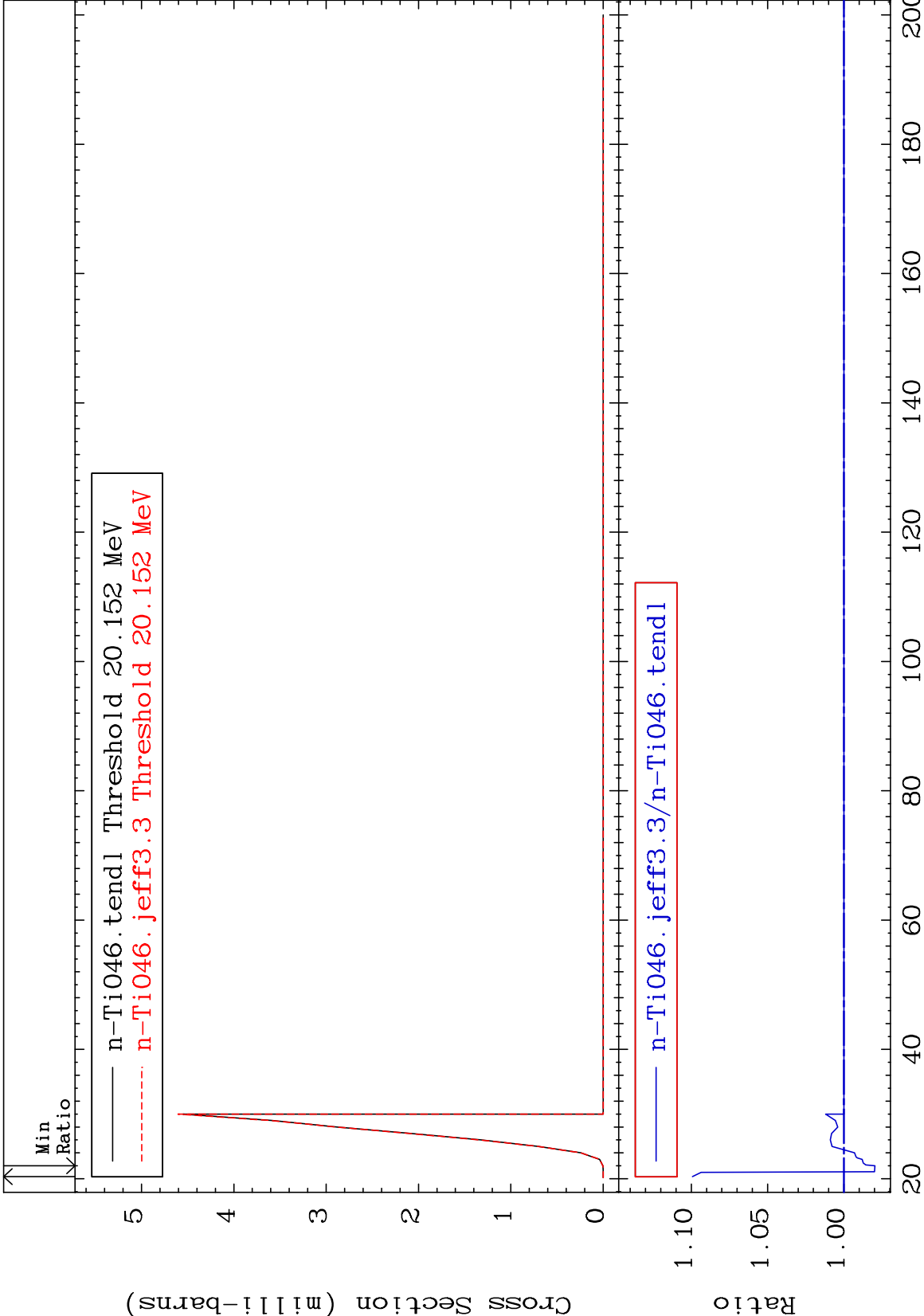
Radionuclide Production Cross Section

-14.79 To 0.000 %



Radionuclide Production Cross Section

-2.032 To 9.925 %



Incident Energy (MeV)

22-Ti-46

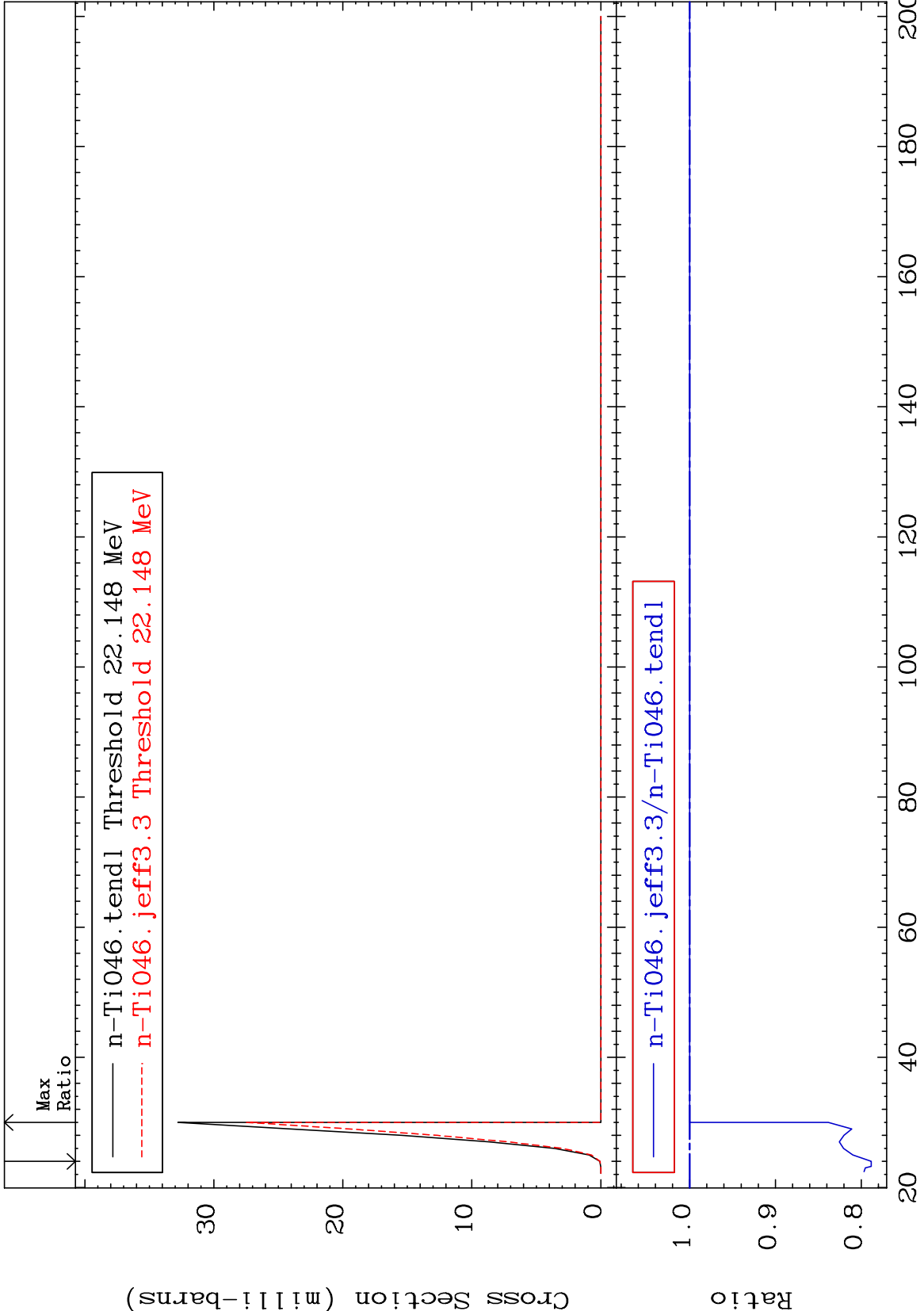
MAT 2225

(n,2n) p:21-Sc-44g

22-Ti-46

Radionuclide Production Cross Section

-21.17 To 0.000 %



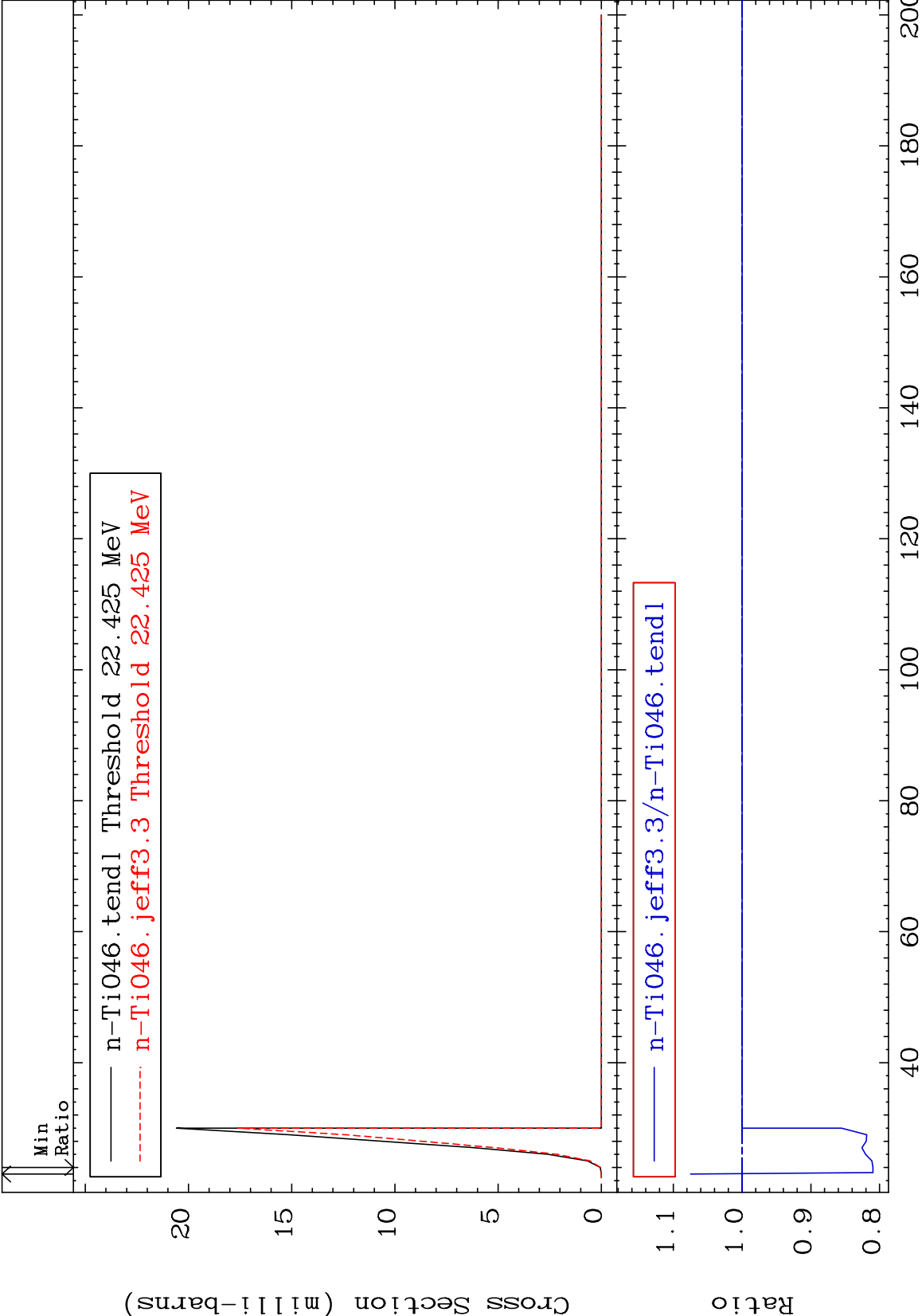
81

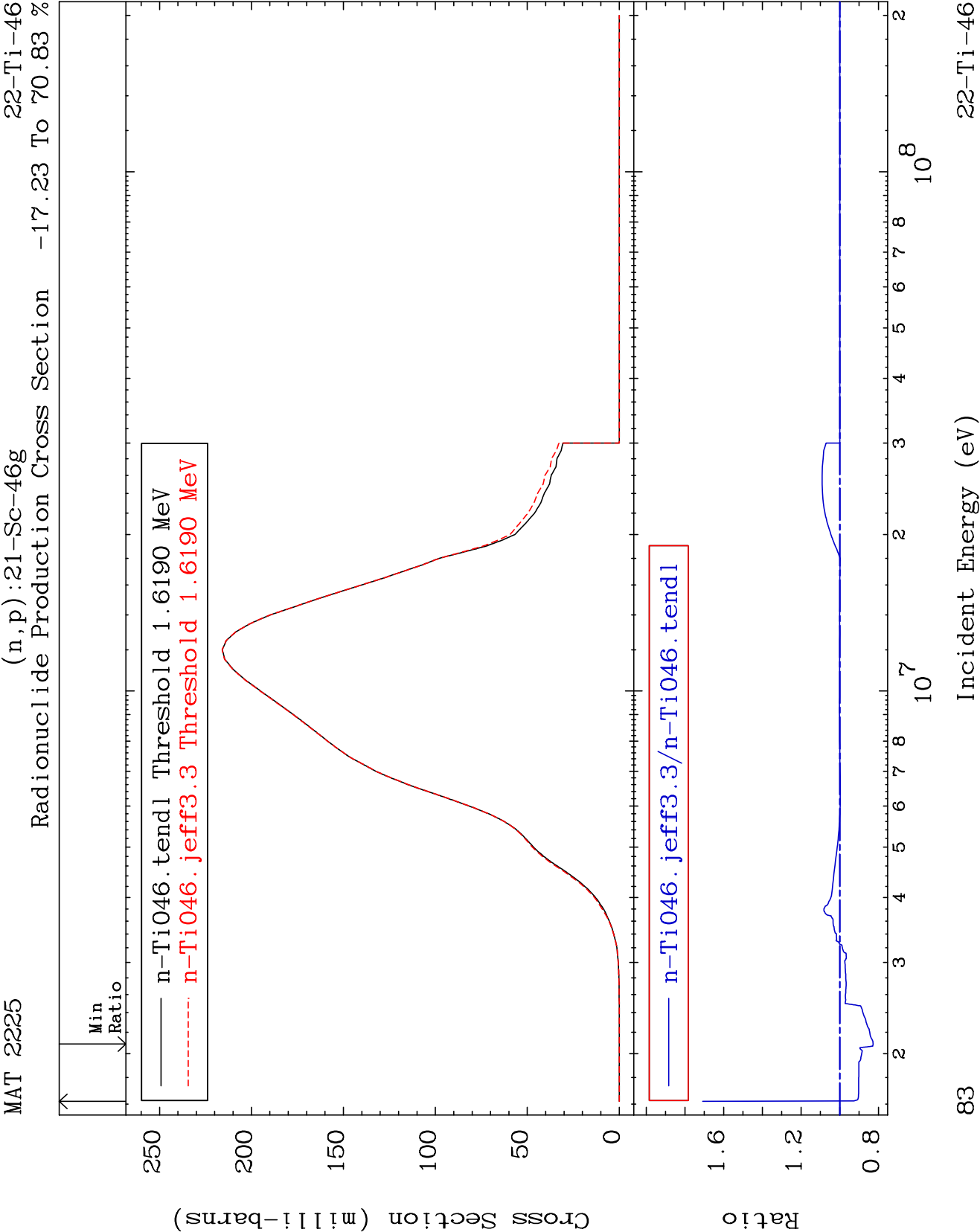
Incident Energy (MeV)

22-Ti-46

Radionuclide Production Cross Section

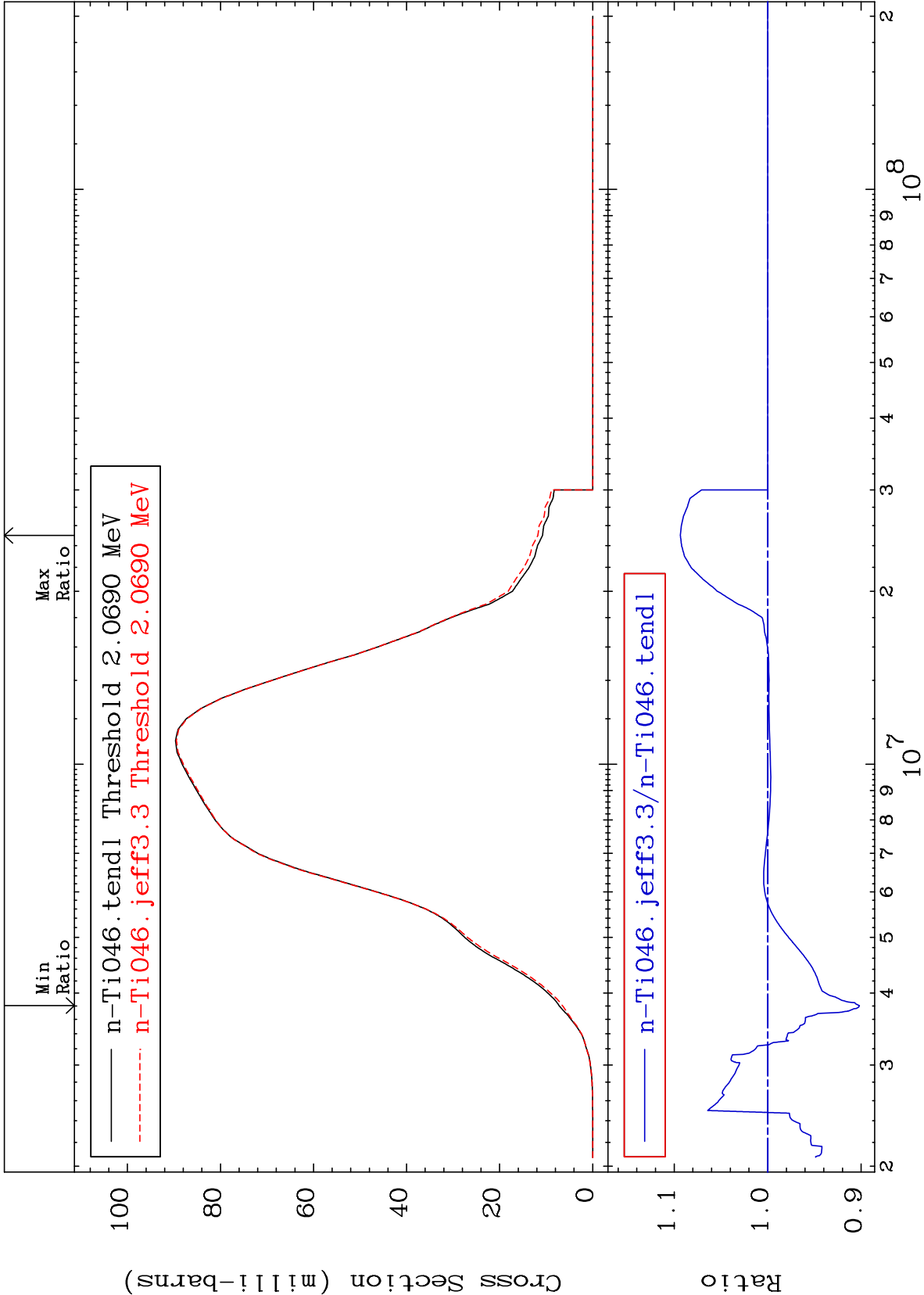
-19.03 To 7.481 %

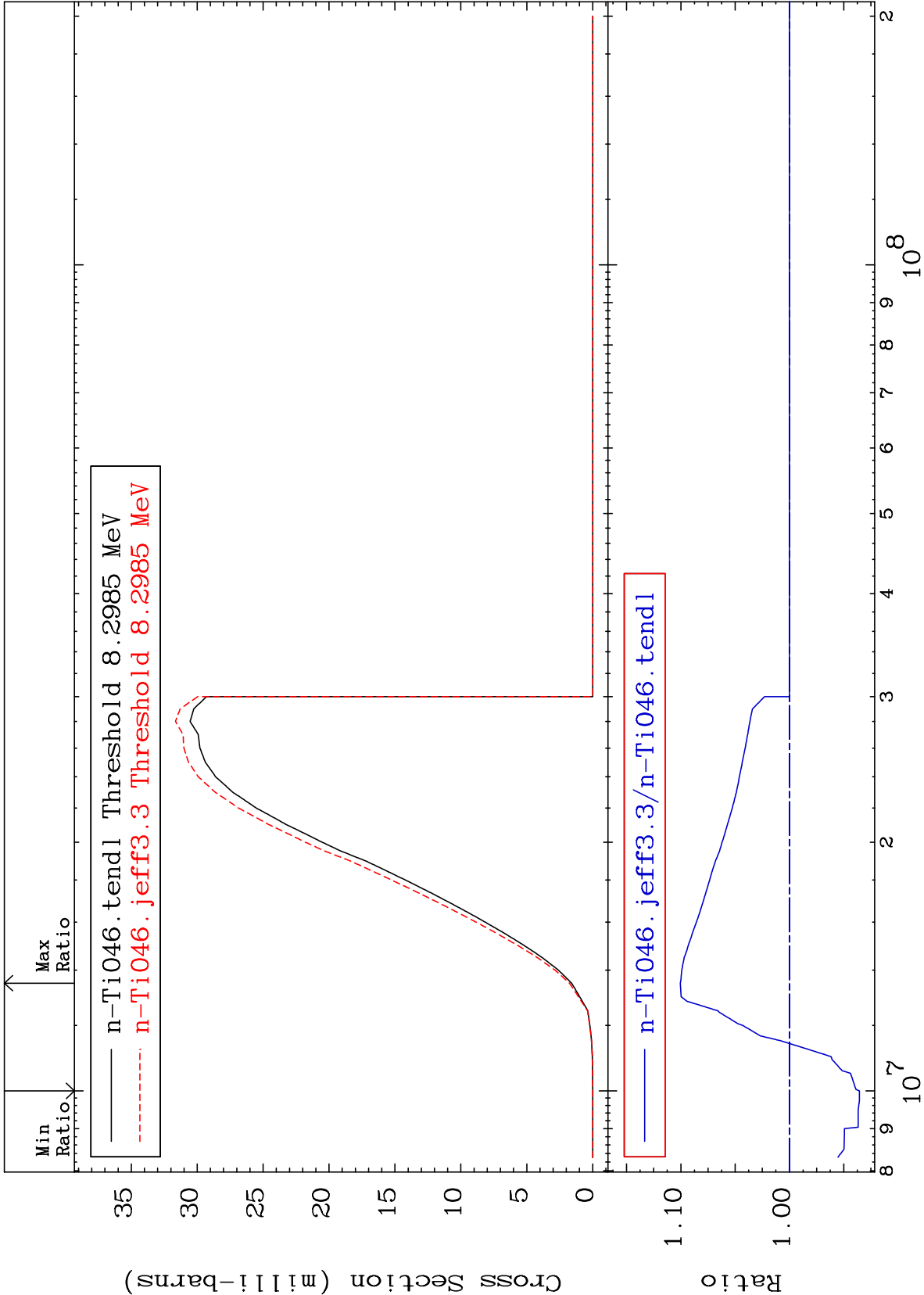




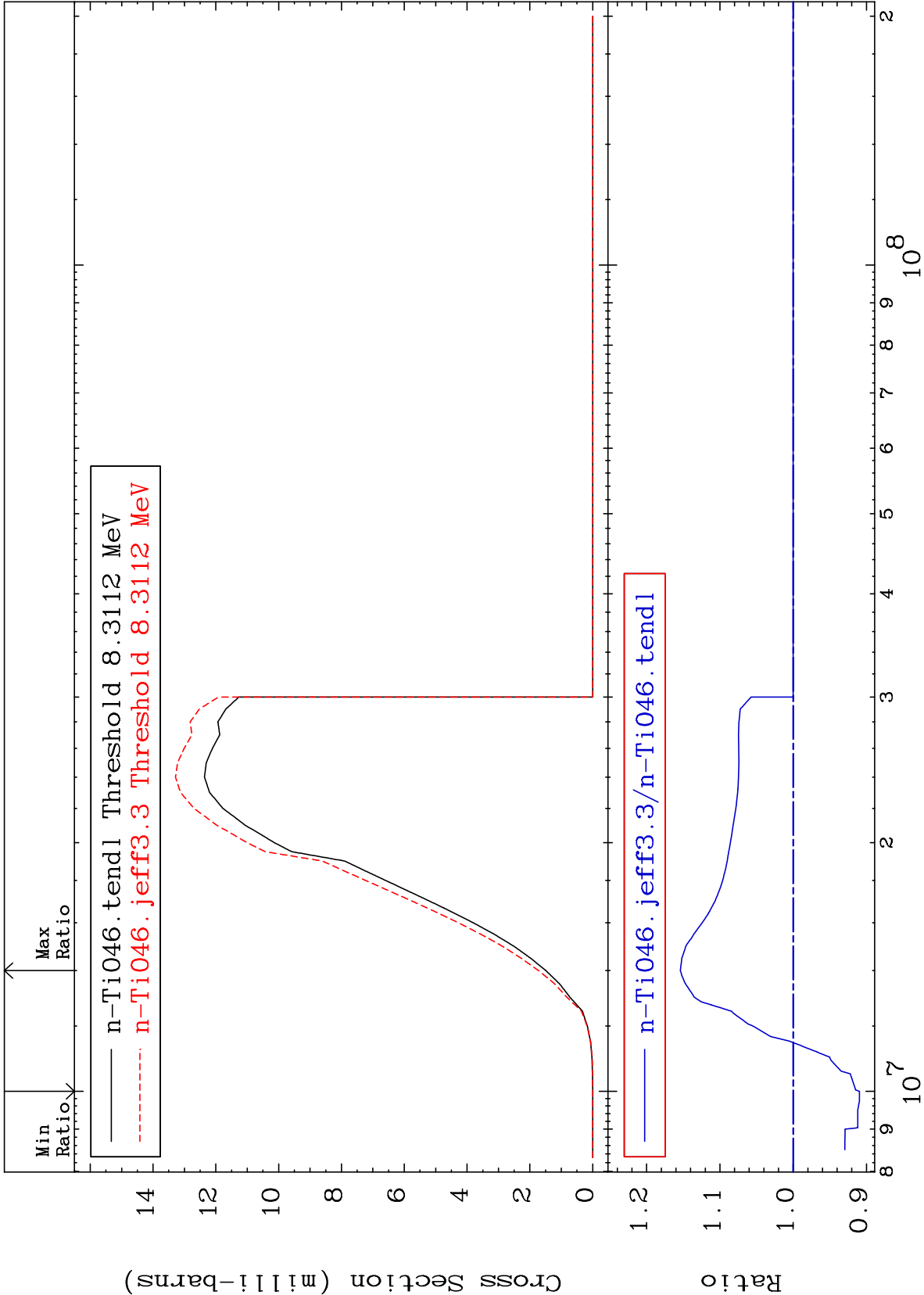
Radionuclide Production Cross Section

-9.813 To 9.343 %





Radionuclide Production Cross Section -9.040 To 15.40 %



Radionuclide Production Cross Section -10.48 To 4.420 %

