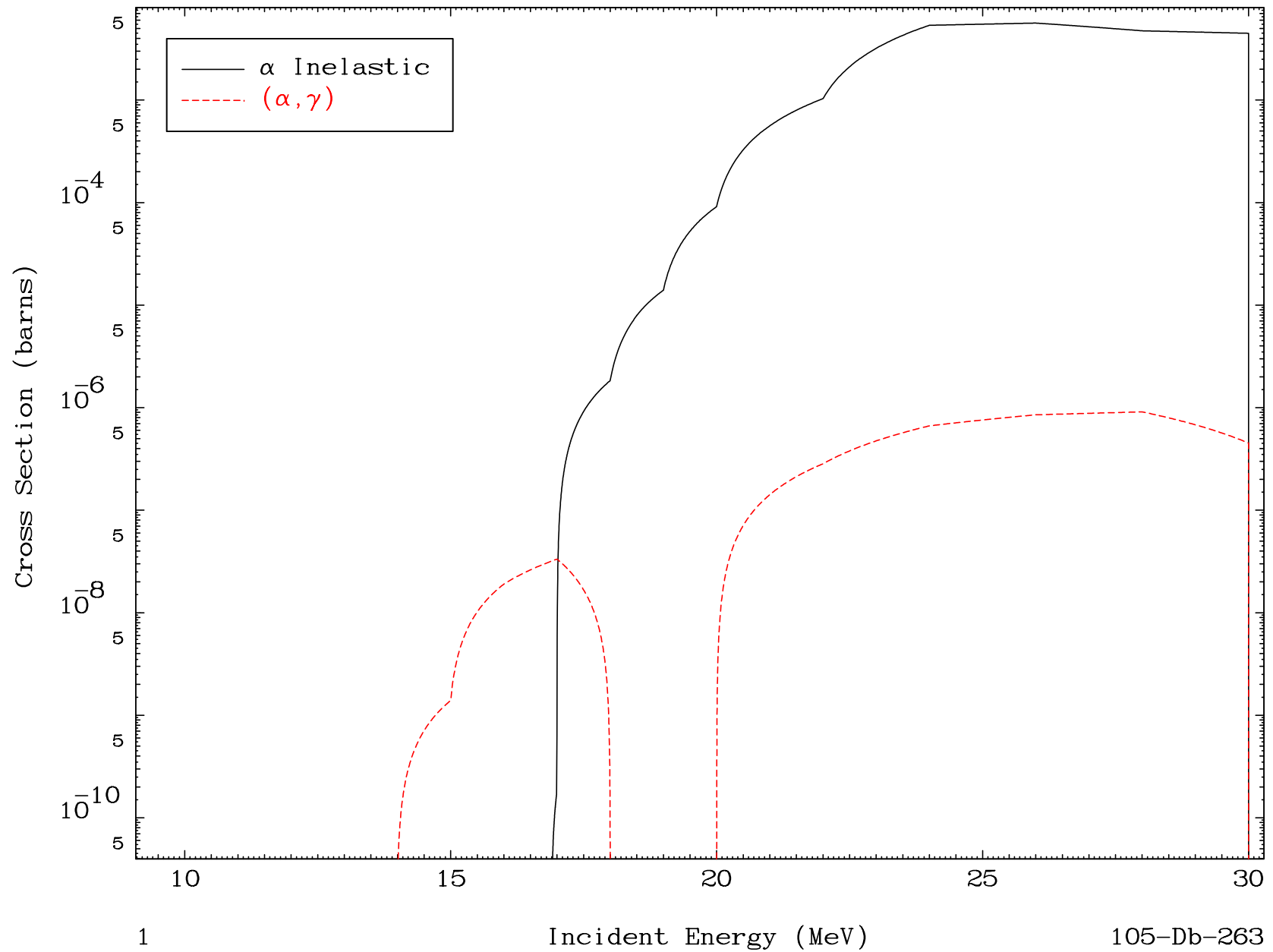
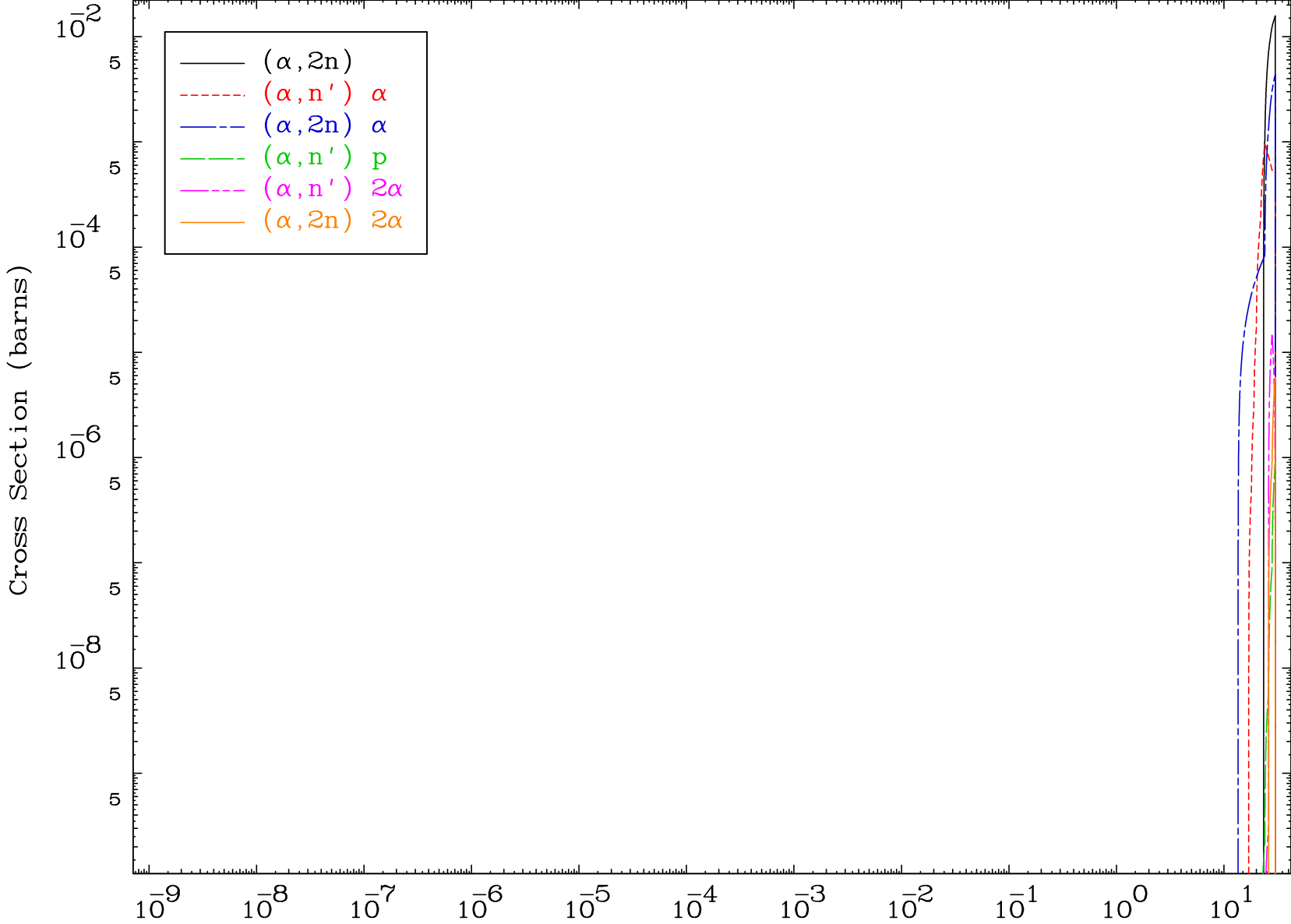
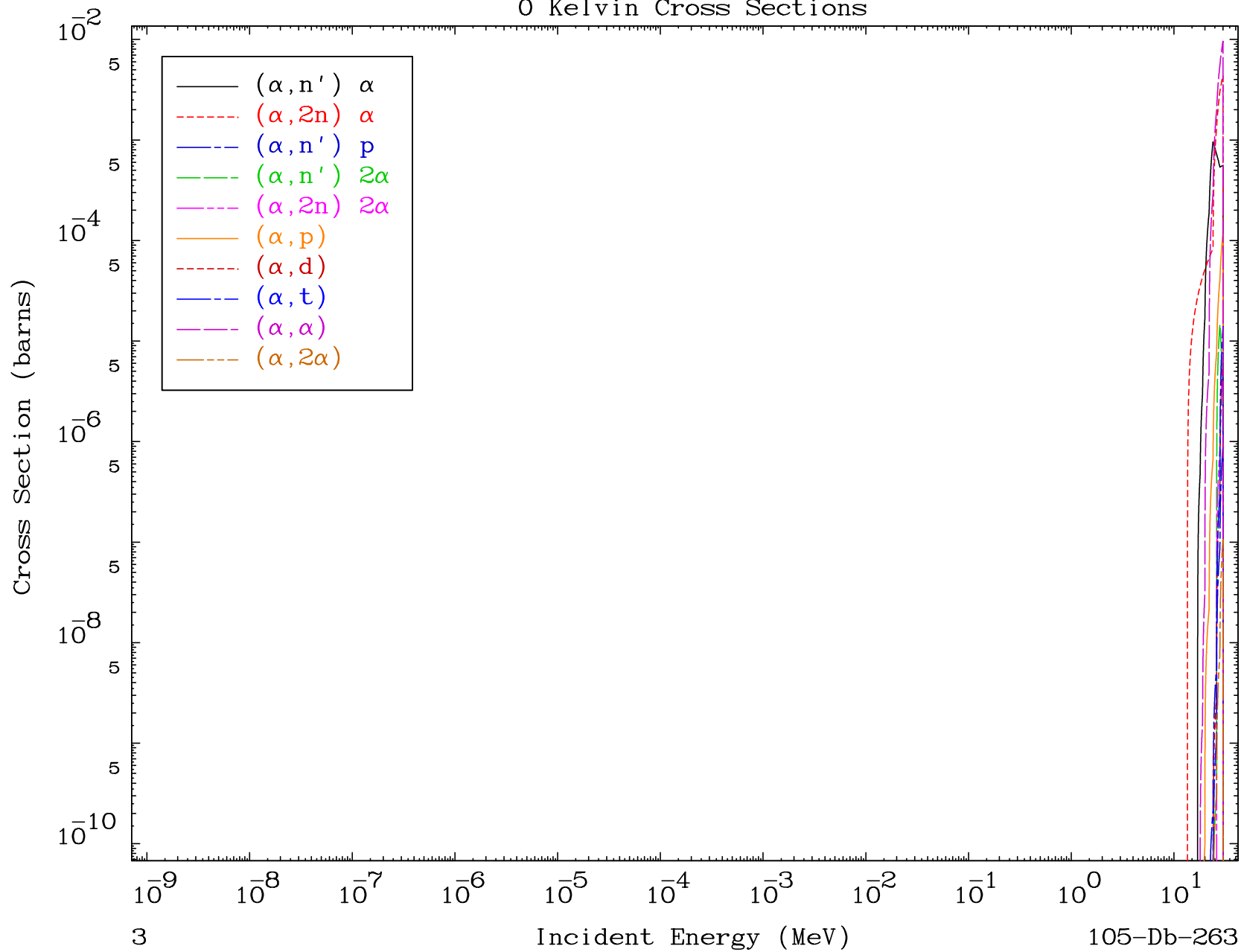
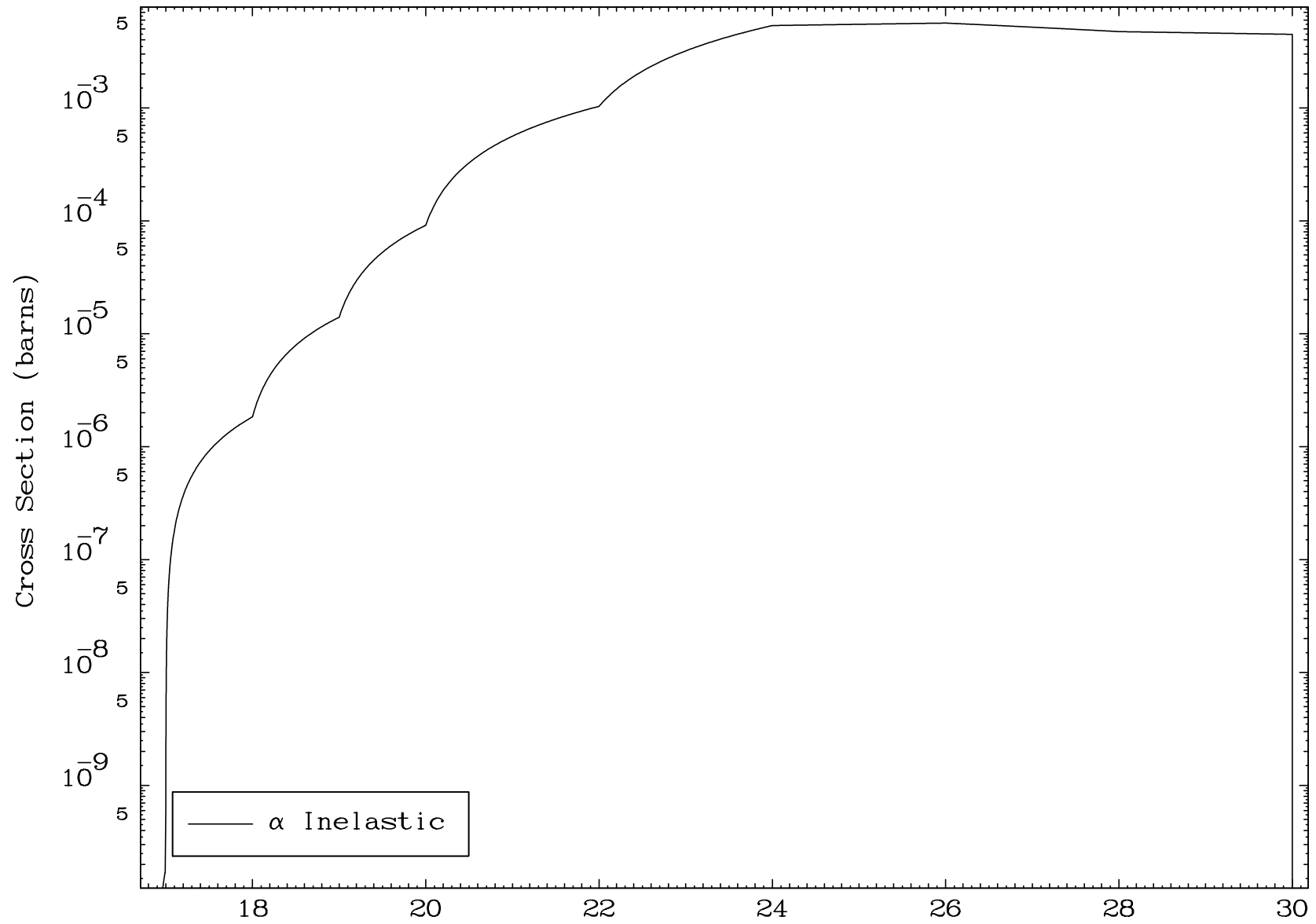






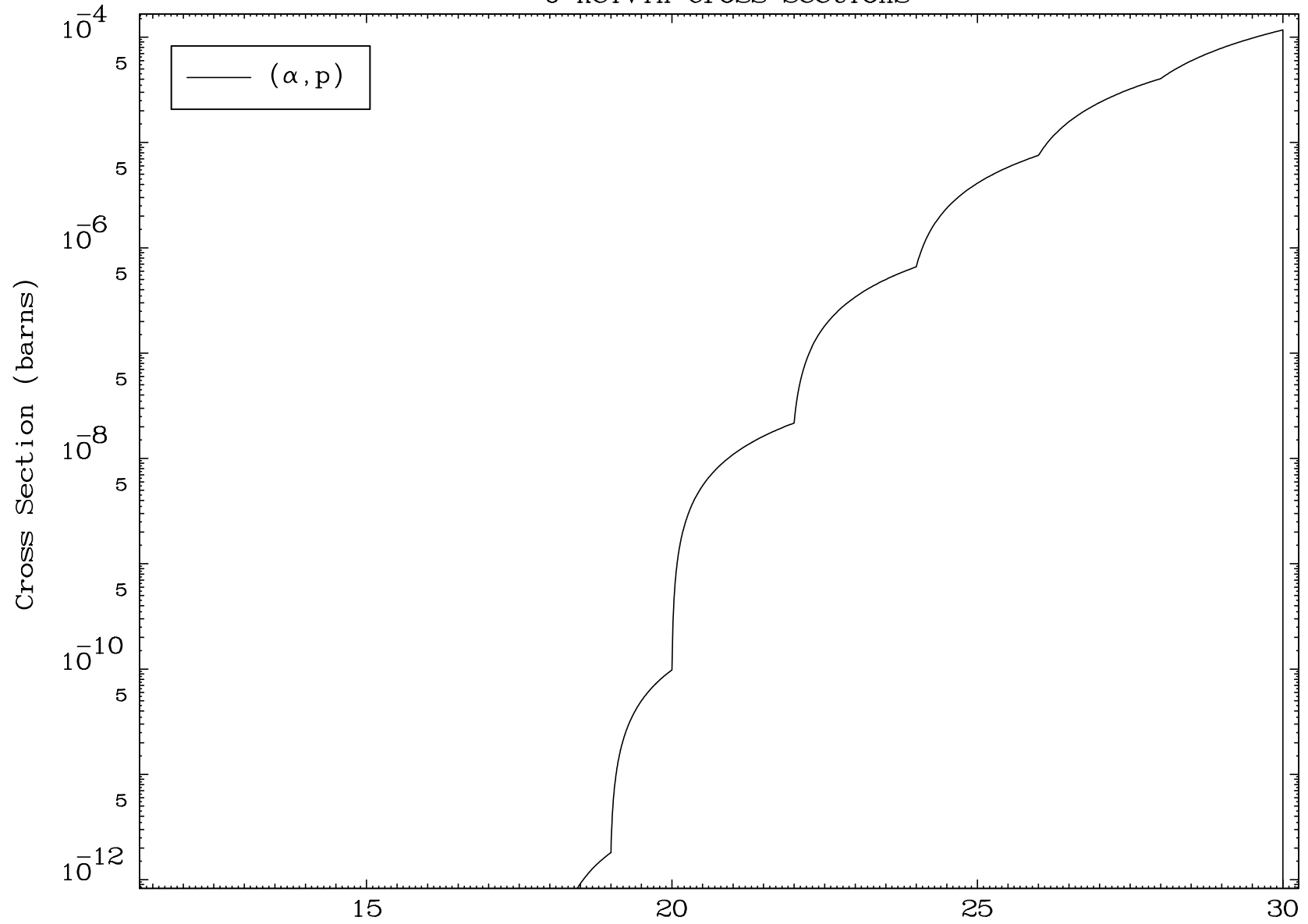




MAT 563

(α ,p) Levels
0 Kelvin Cross Sections

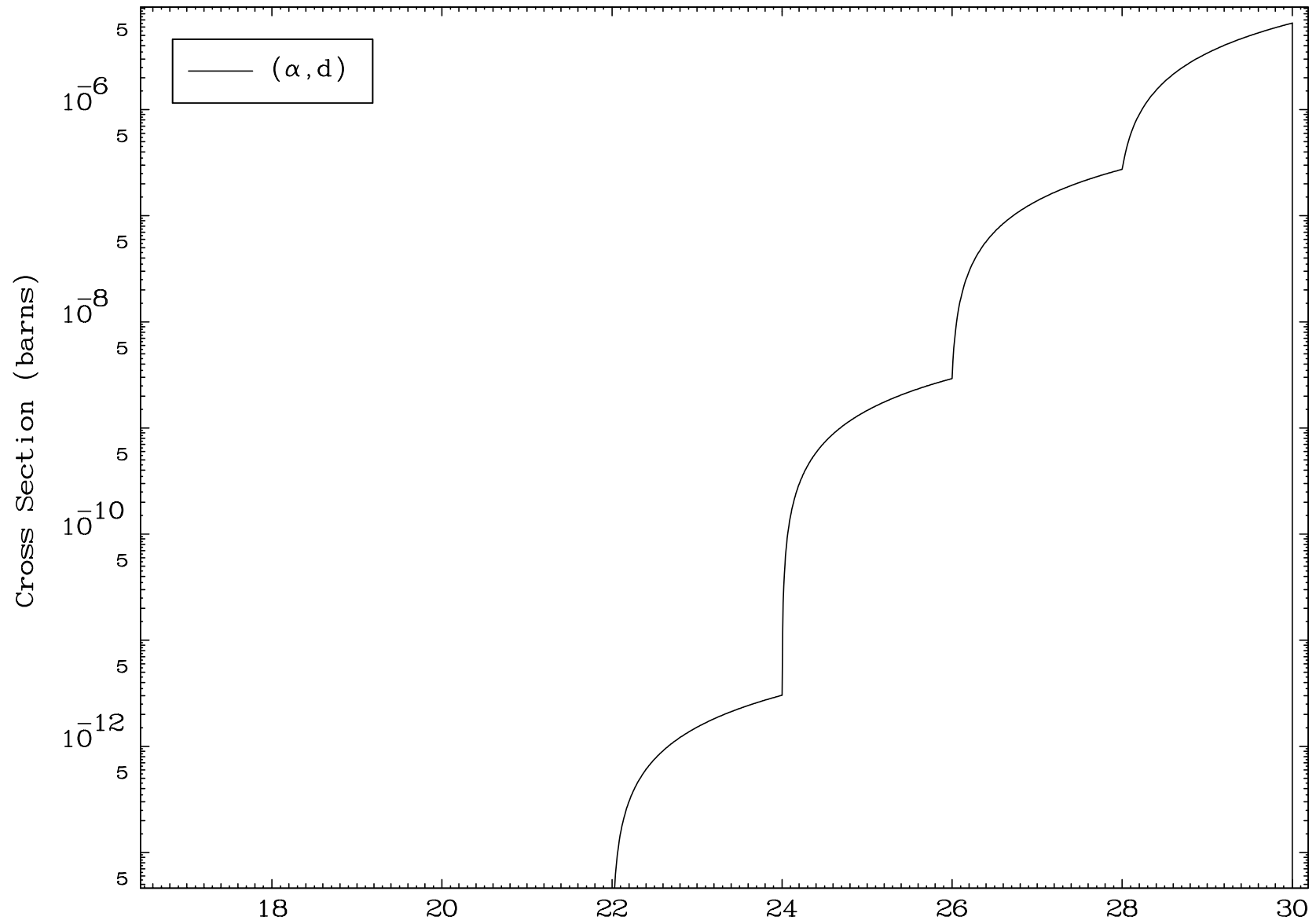
105-Db-263

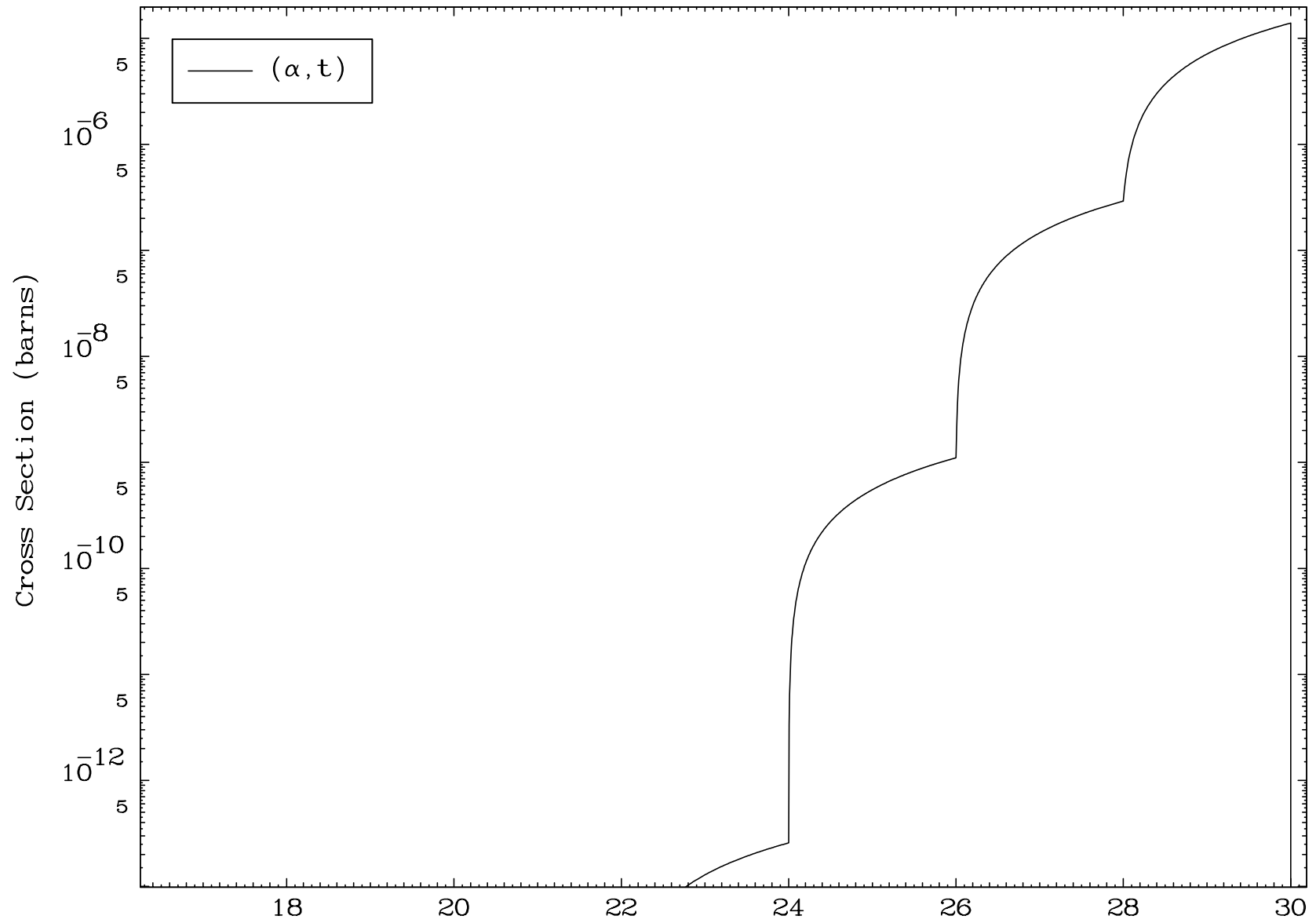


5

Incident Energy (MeV)

105-Db-263

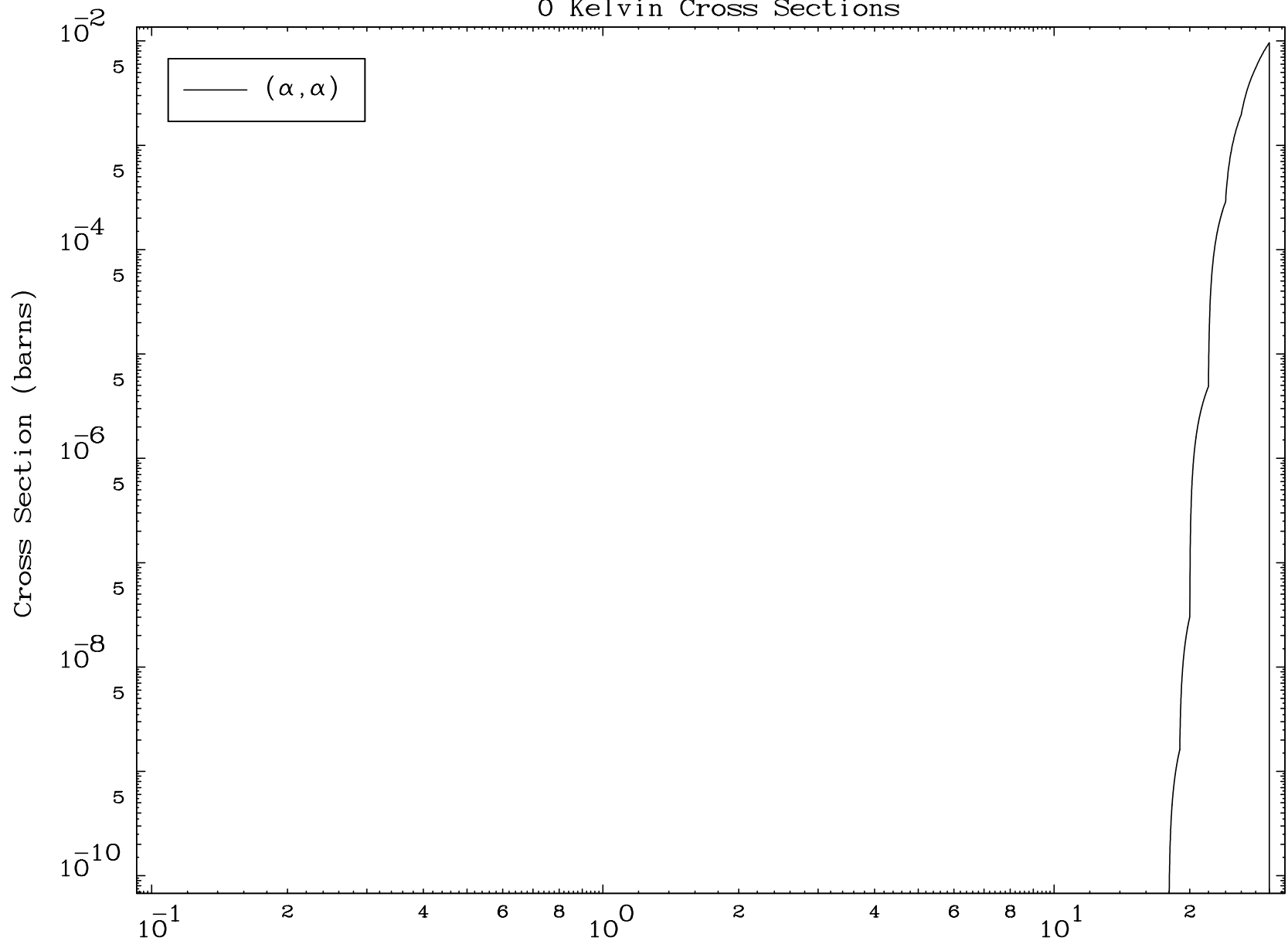




MAT 563

(α, α) Levels
0 Kelvin Cross Sections

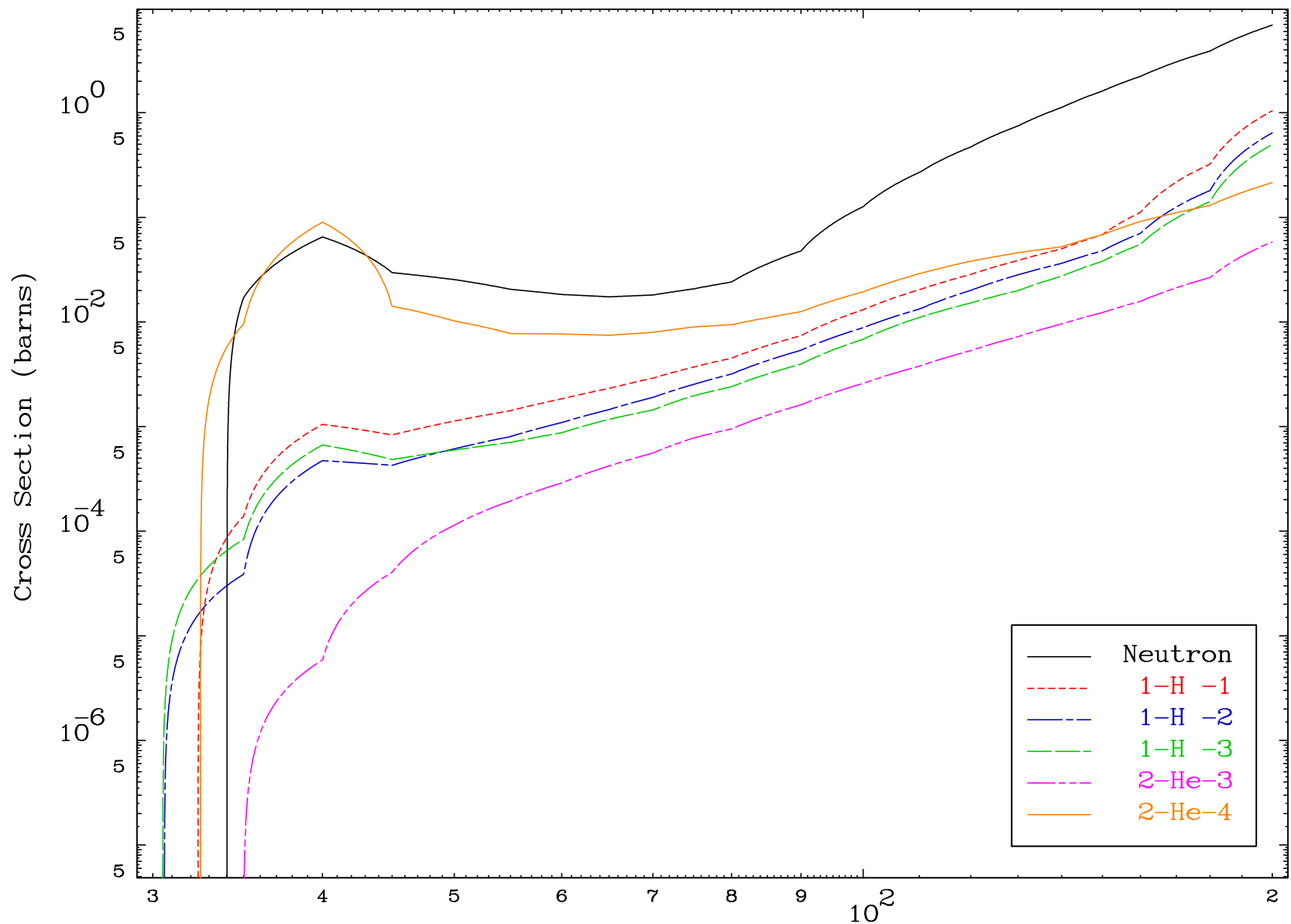
105-Db-263



8

Incident Energy (MeV)

105-Db-263

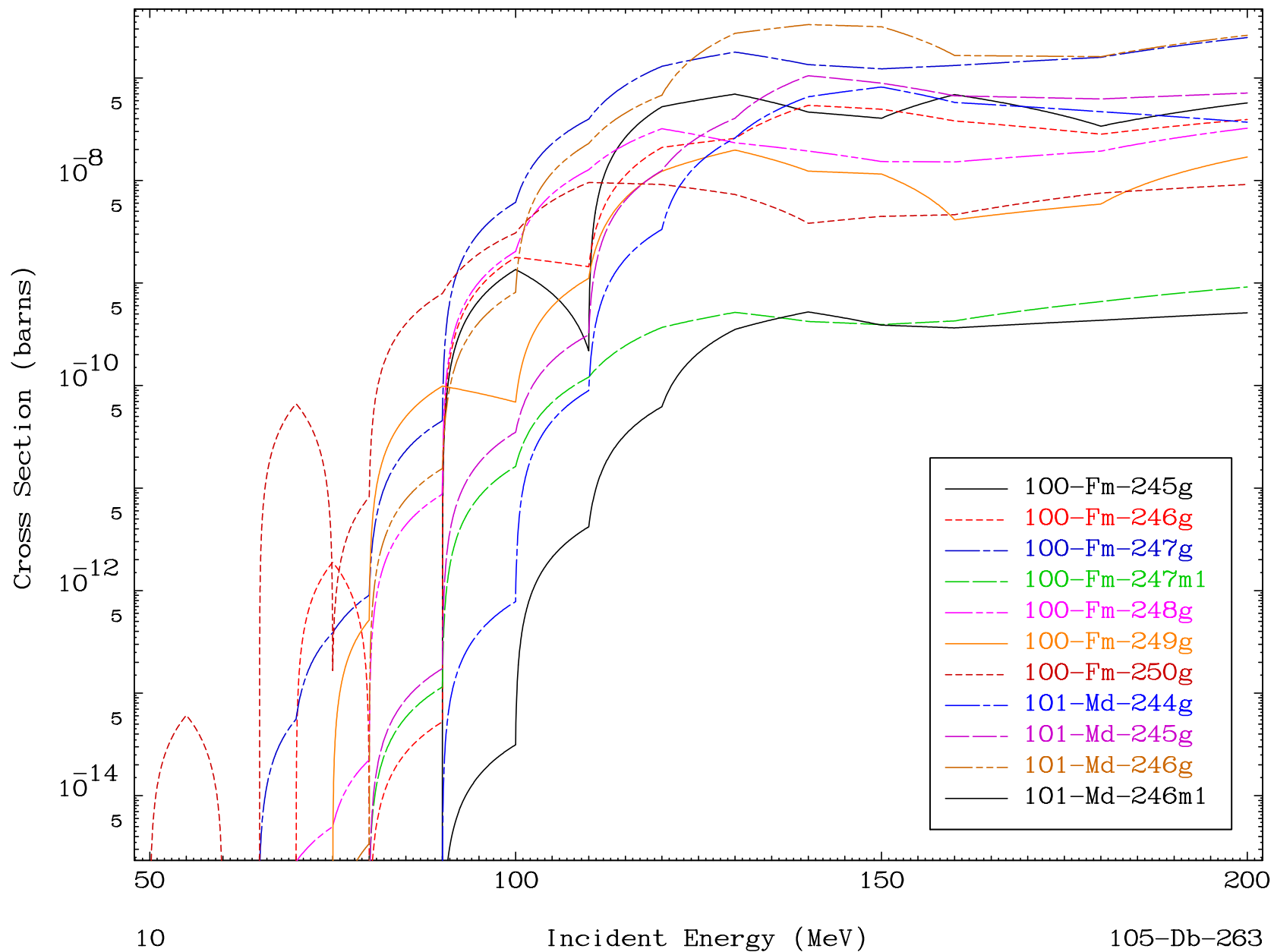


MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section

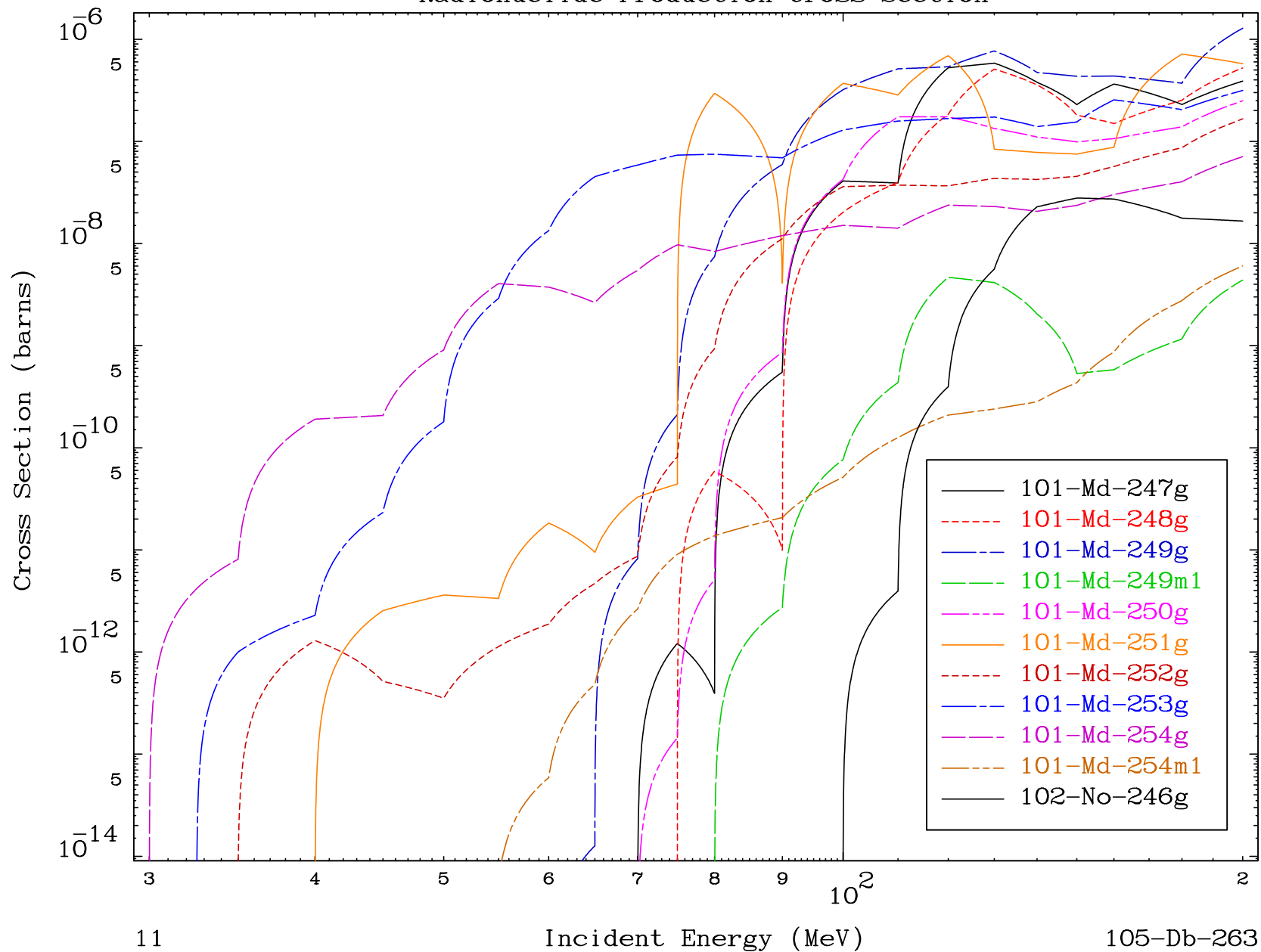


MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section

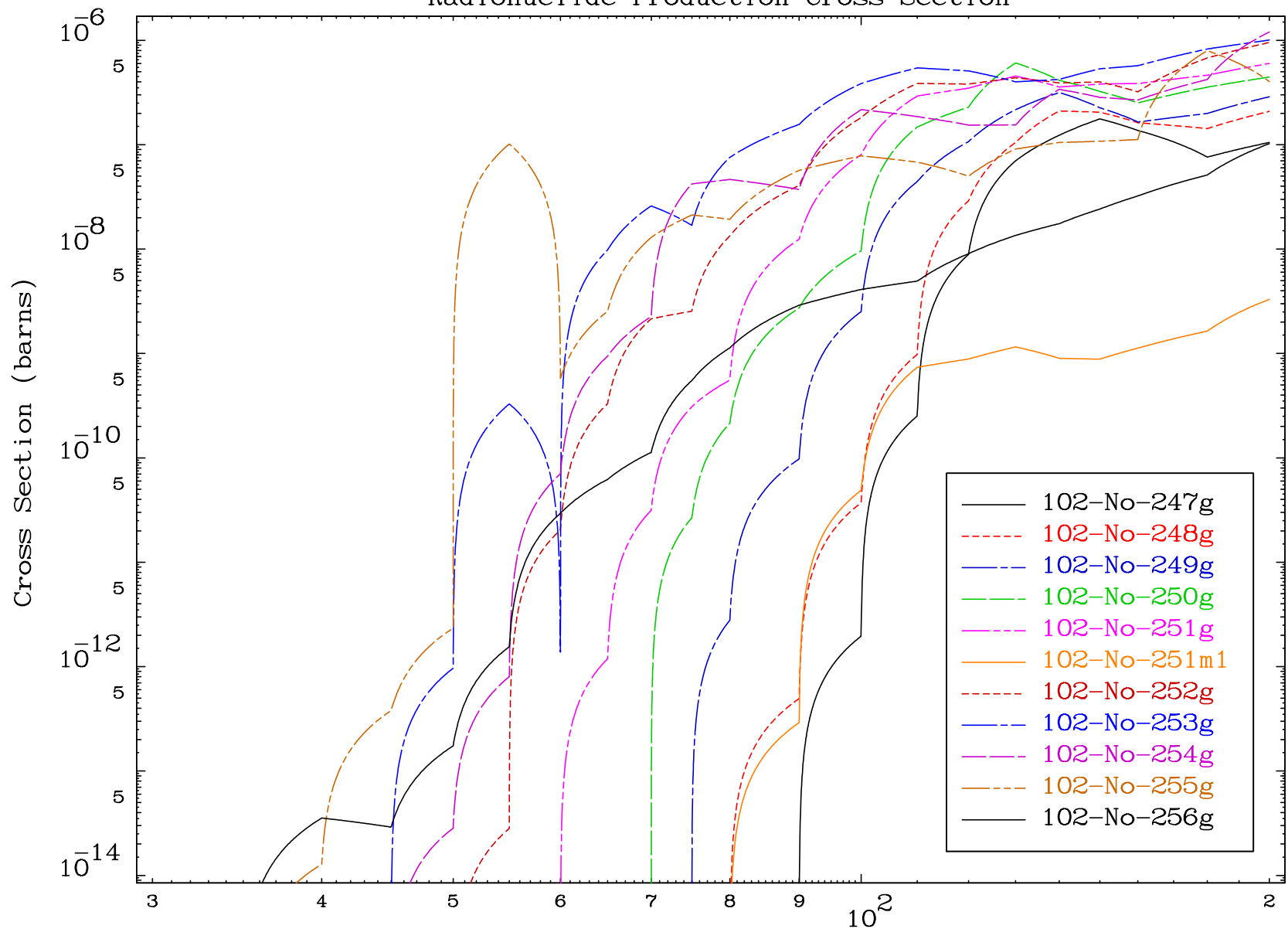


MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



12

Incident Energy (MeV)

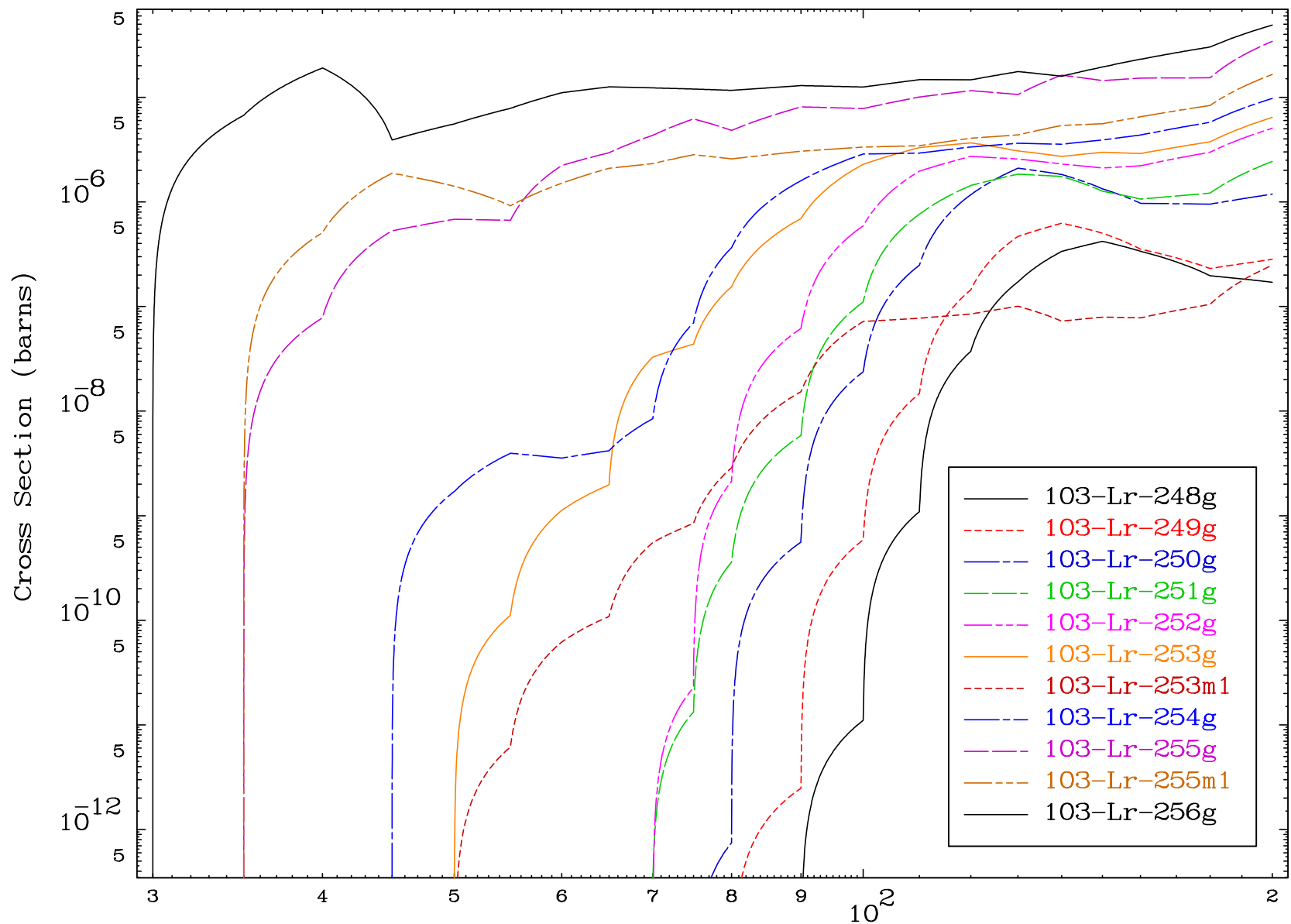
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



13

Incident Energy (MeV)

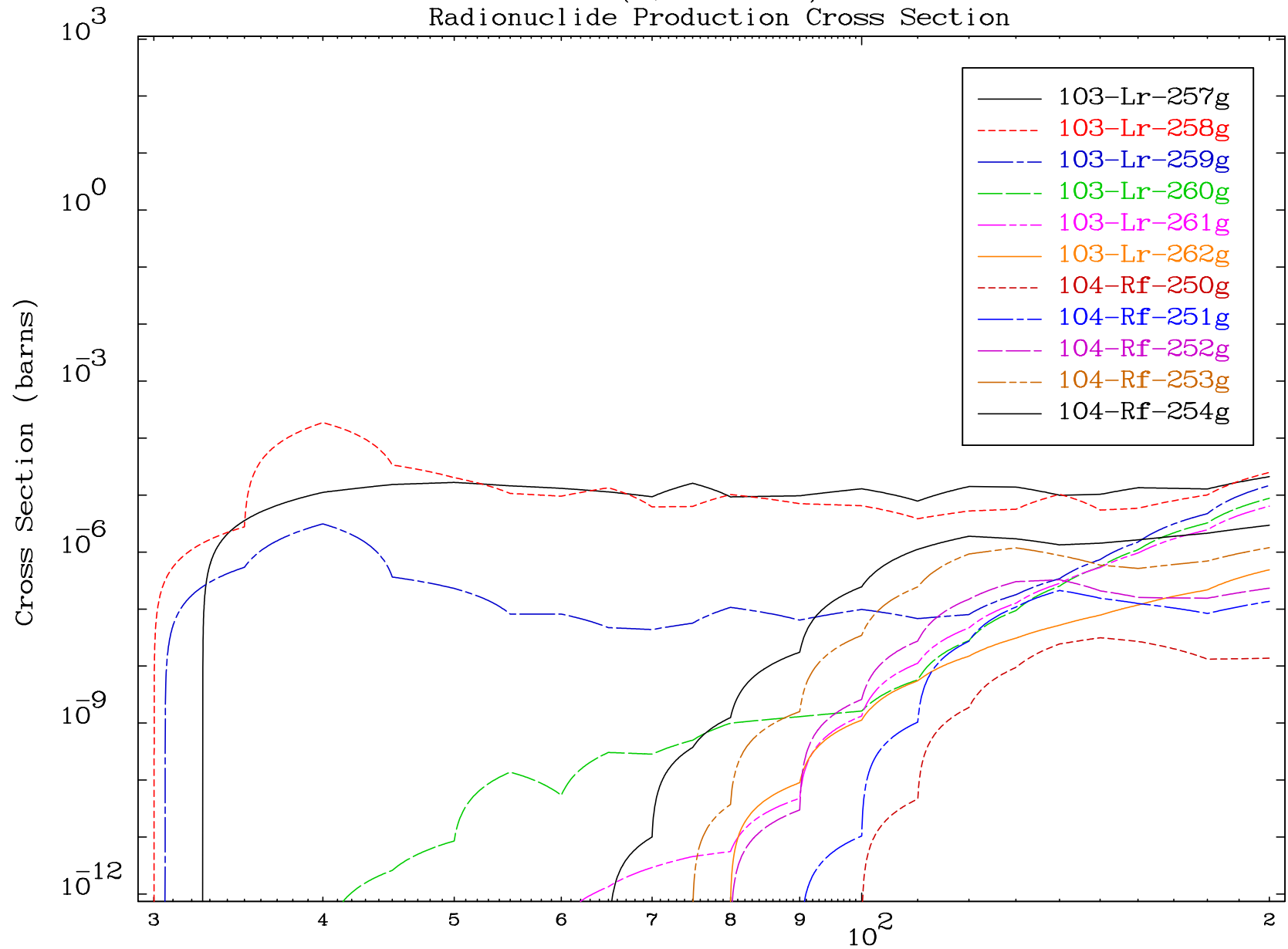
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



14

Incident Energy (MeV)

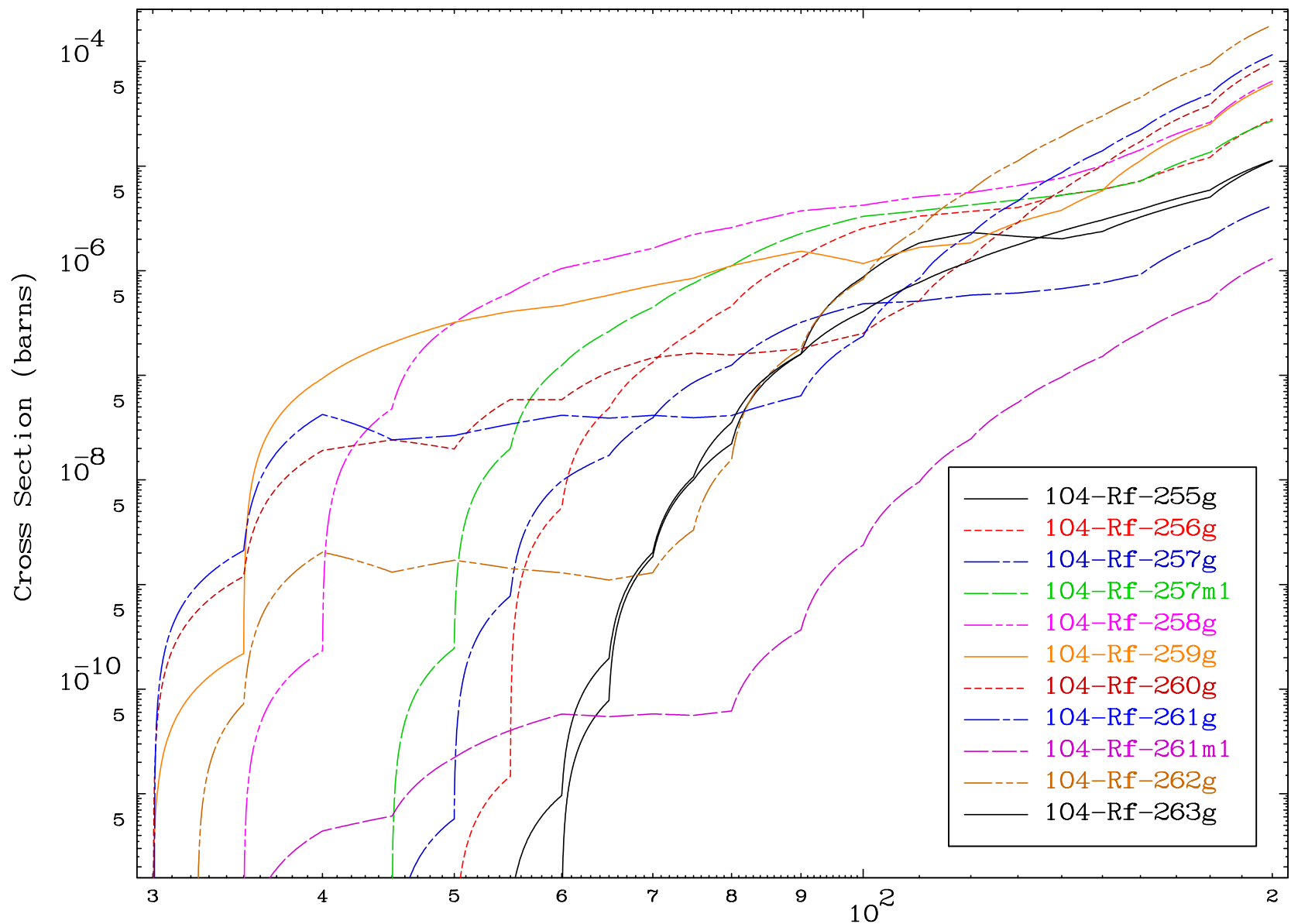
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



15

Incident Energy (MeV)

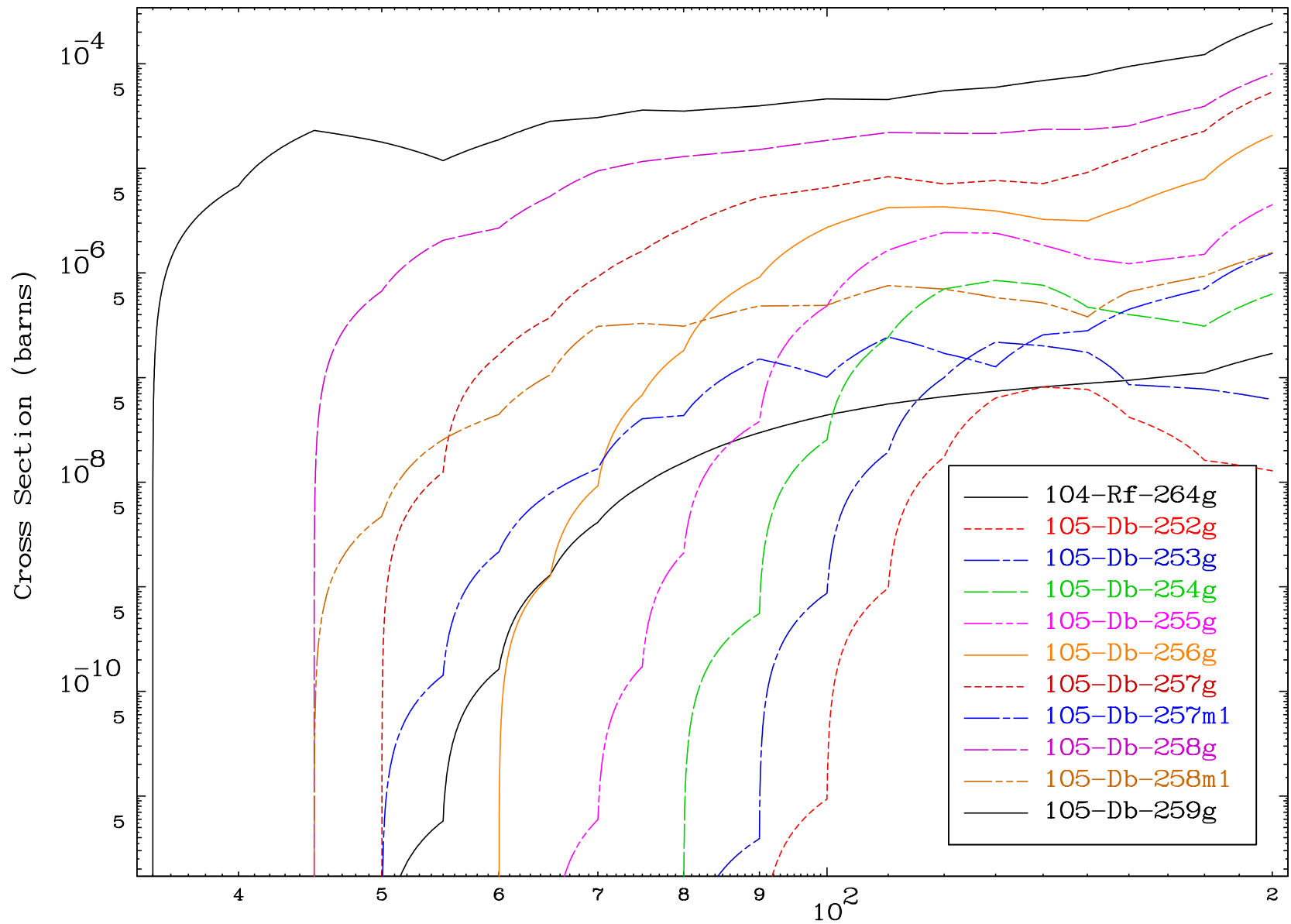
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



16

Incident Energy (MeV)

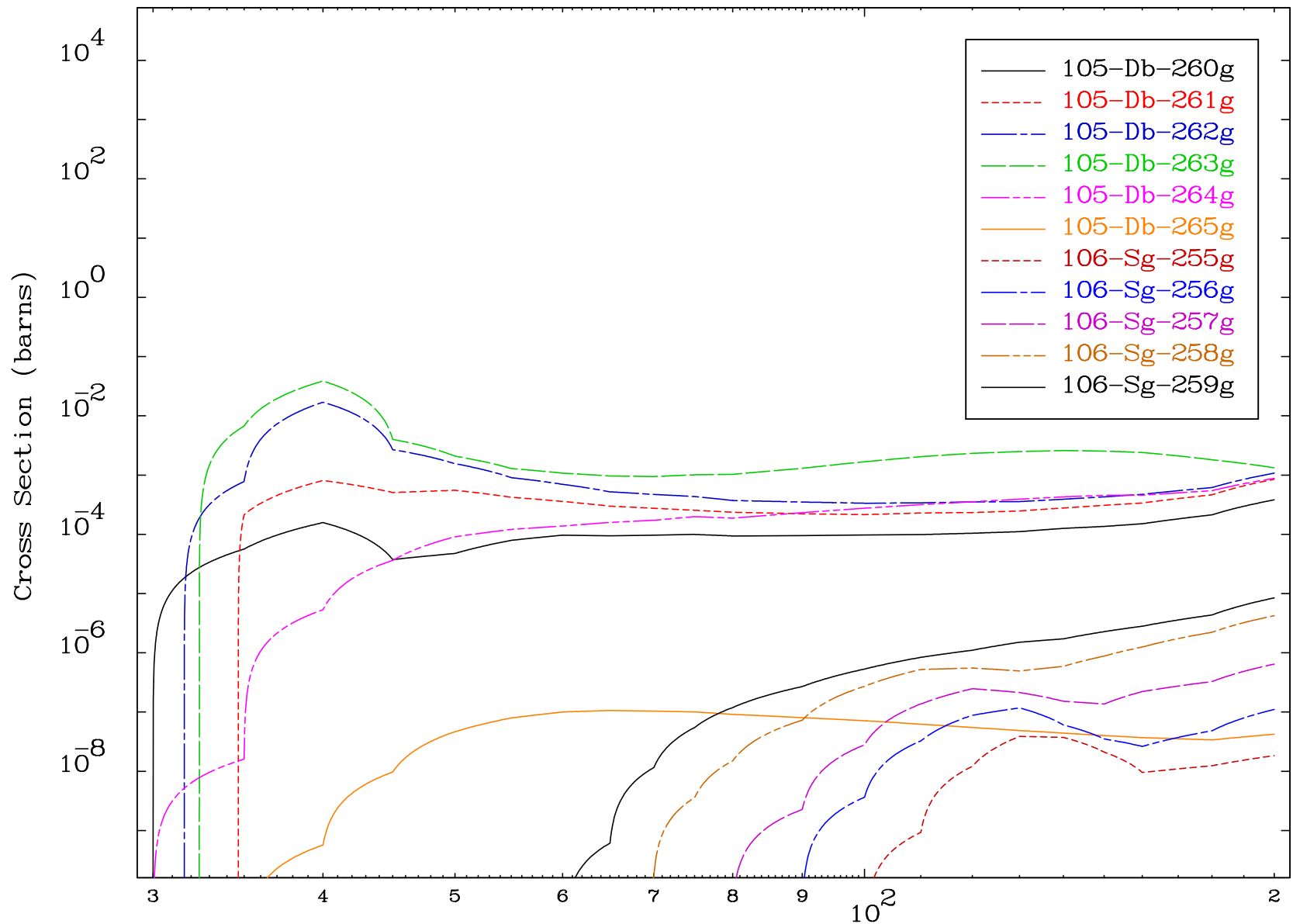
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section



17

Incident Energy (MeV)

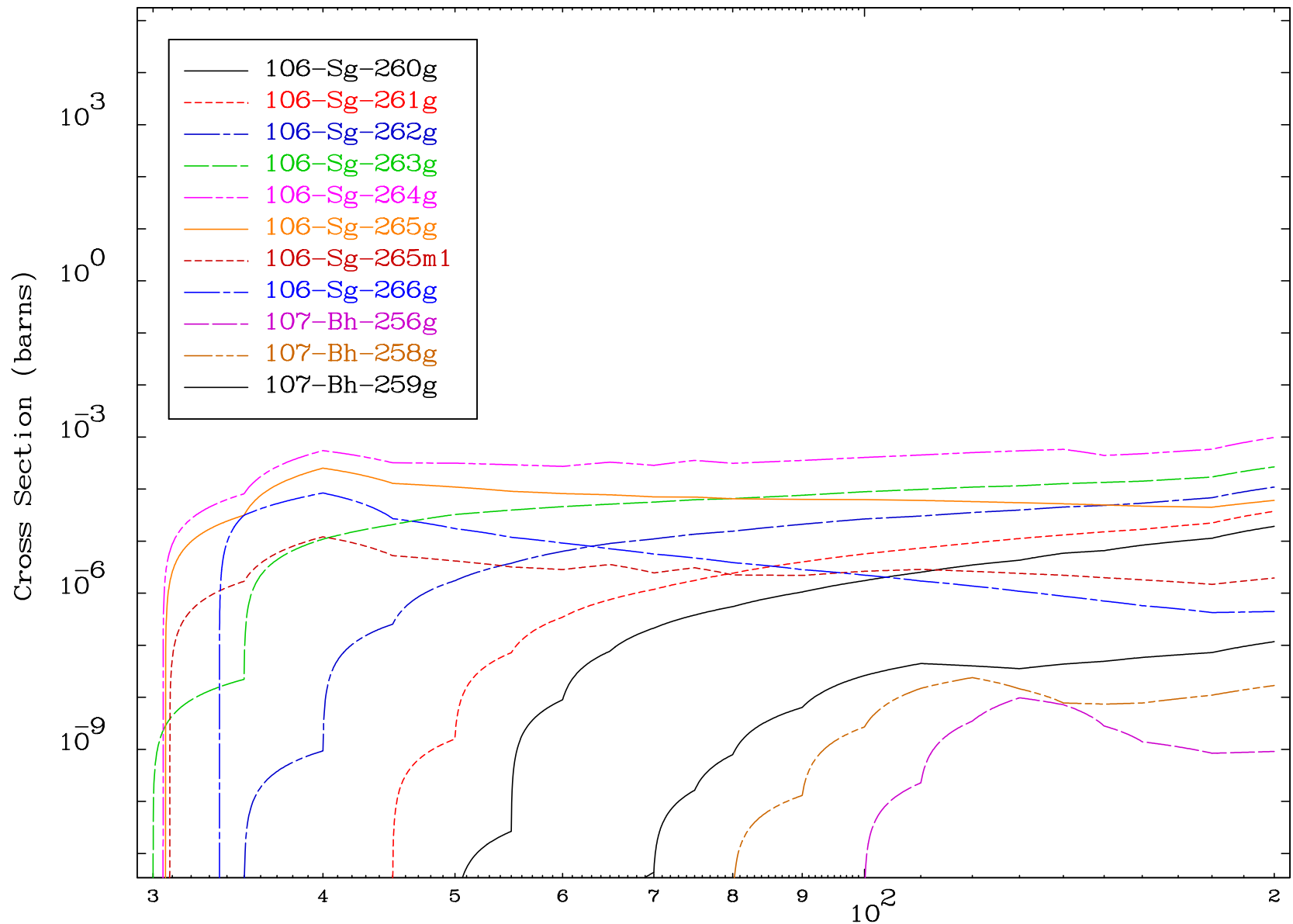
105-Db-263

MAT 563

 $(\alpha, \text{remainder})$

105-Db-263

Radionuclide Production Cross Section

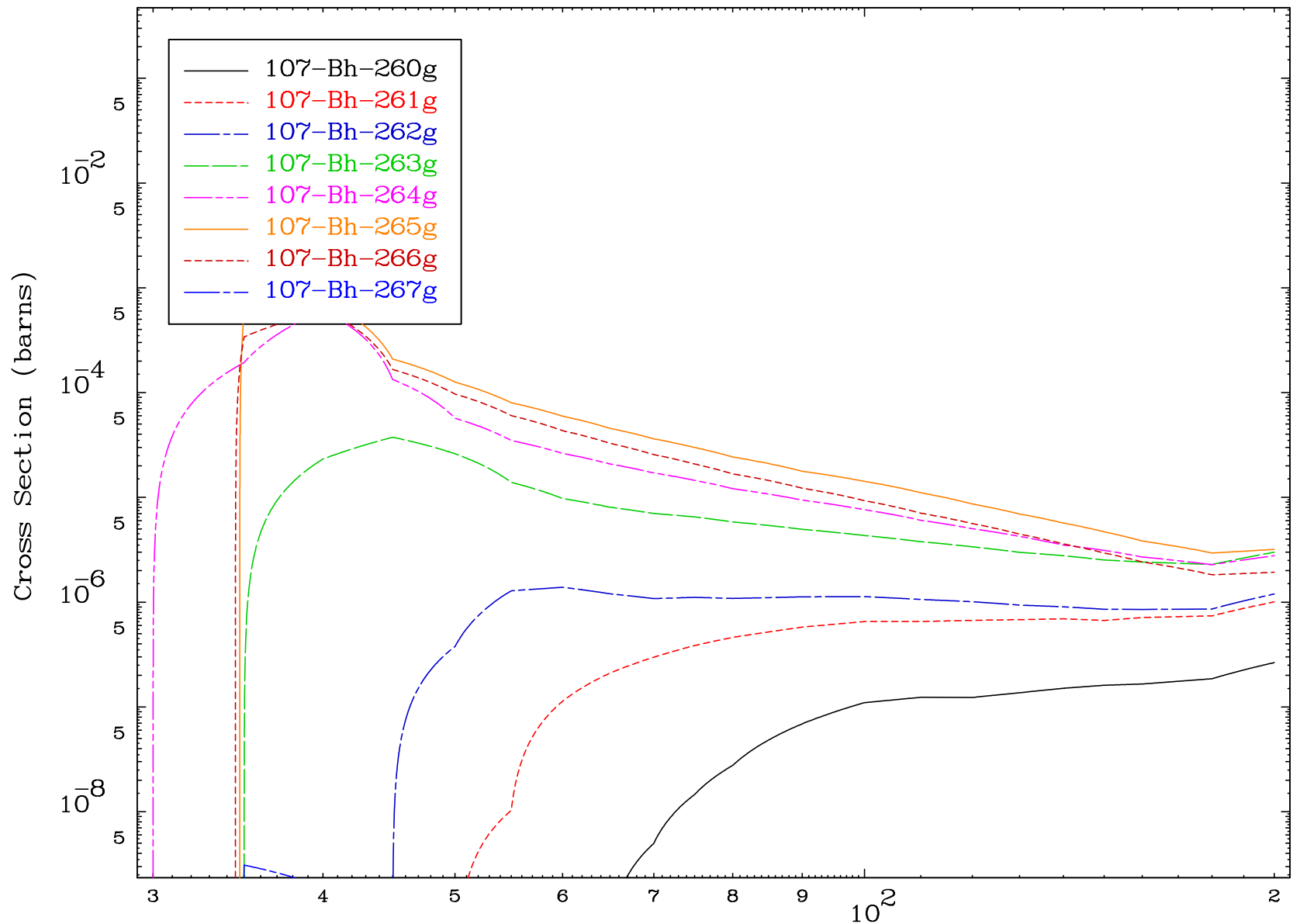


18

Incident Energy (MeV)

105-Db-263

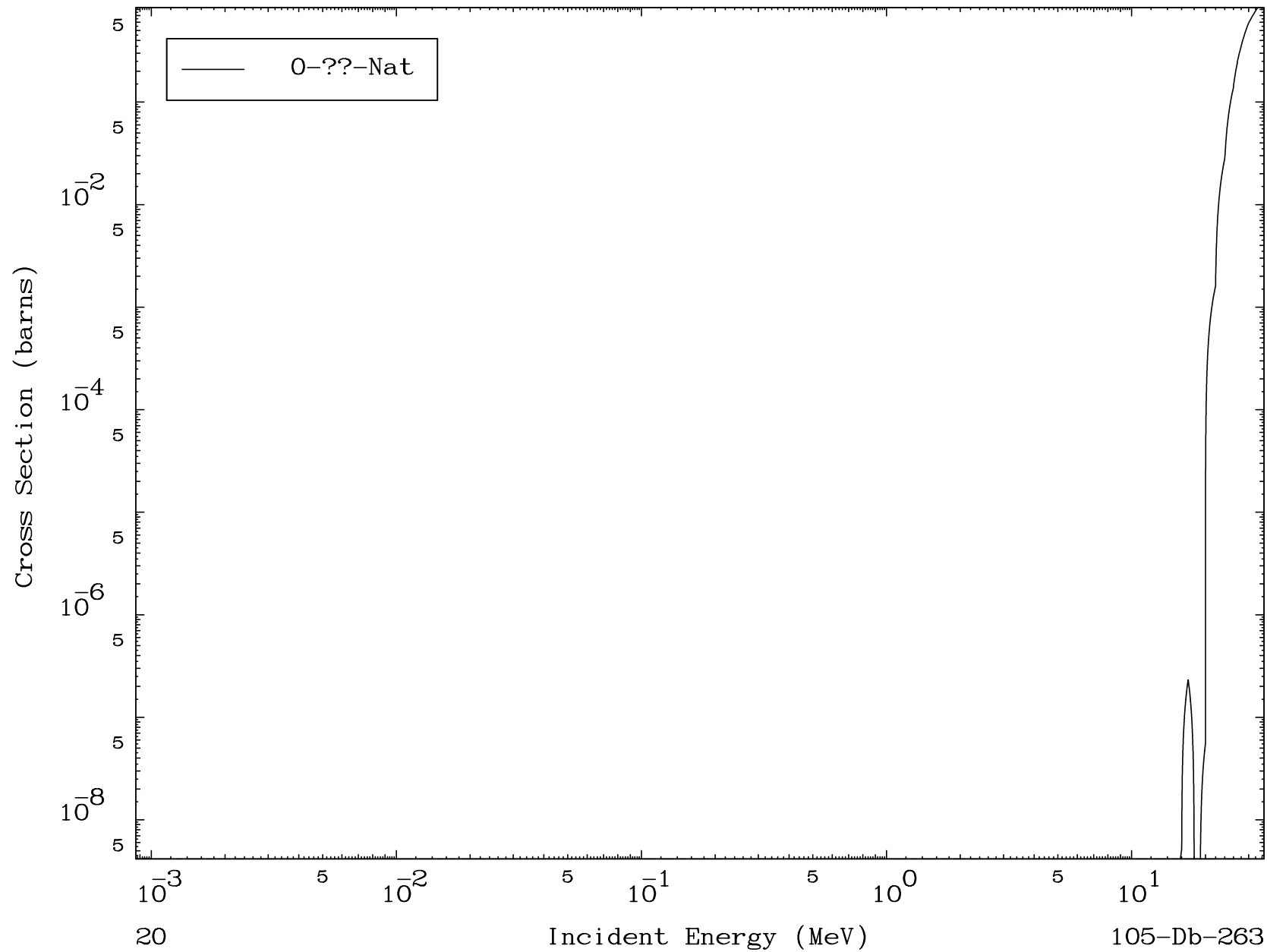
Radionuclide Production Cross Section

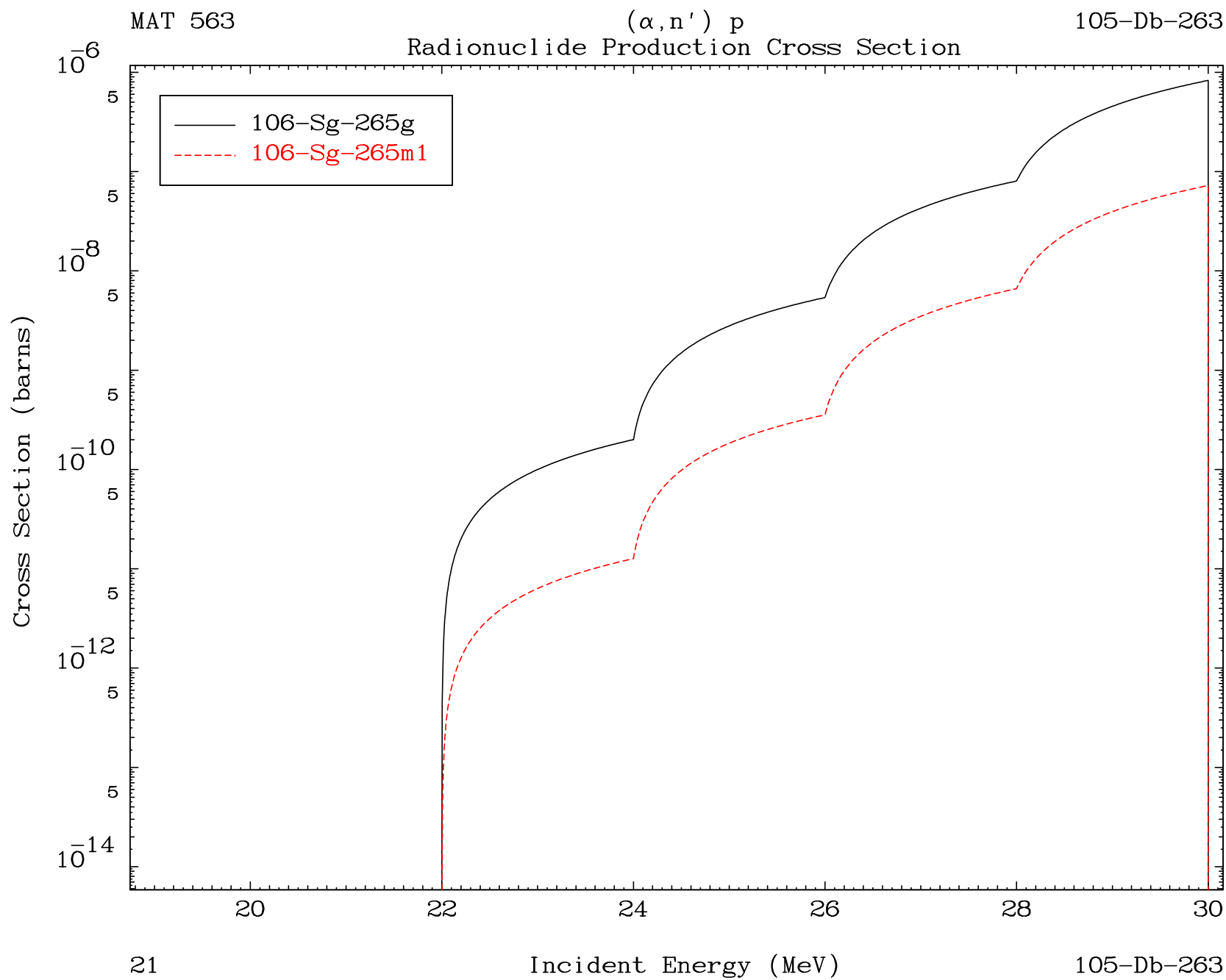


MAT 563

α Fission
Radionuclide Production Cross Section

105-Db-263



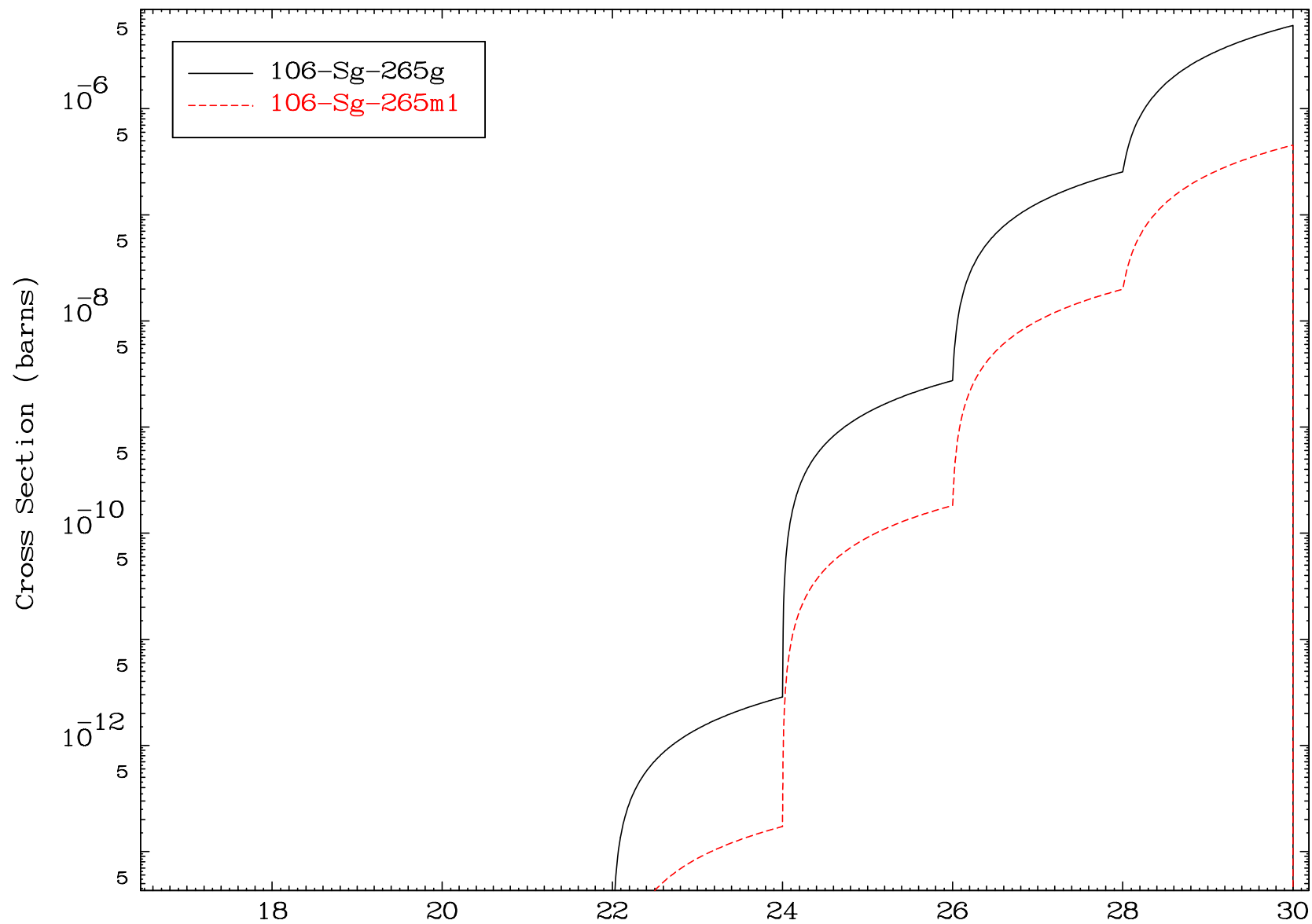


MAT 563

 (α, d)

105-Db-263

Radionuclide Production Cross Section



22

Incident Energy (MeV)

105-Db-263