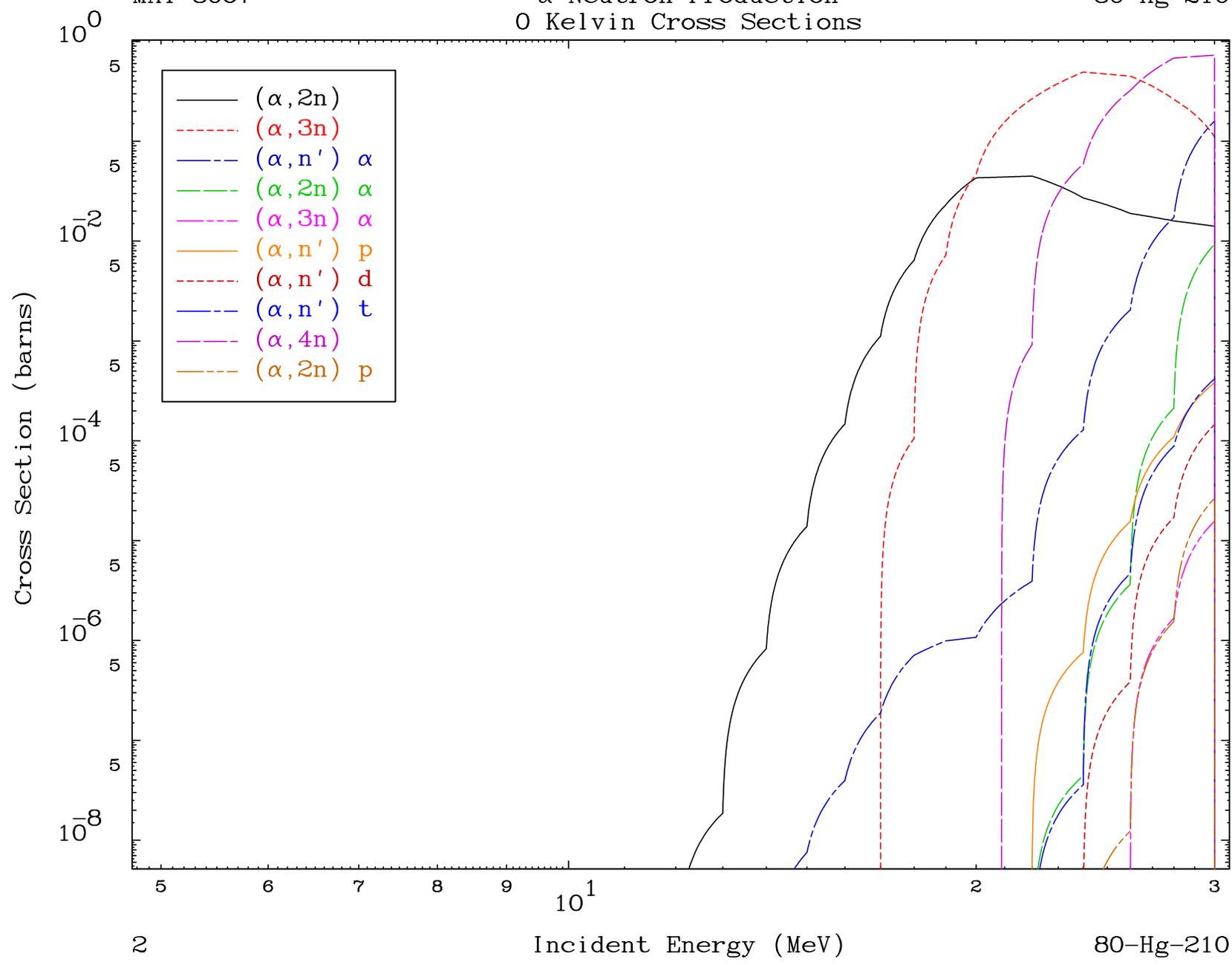
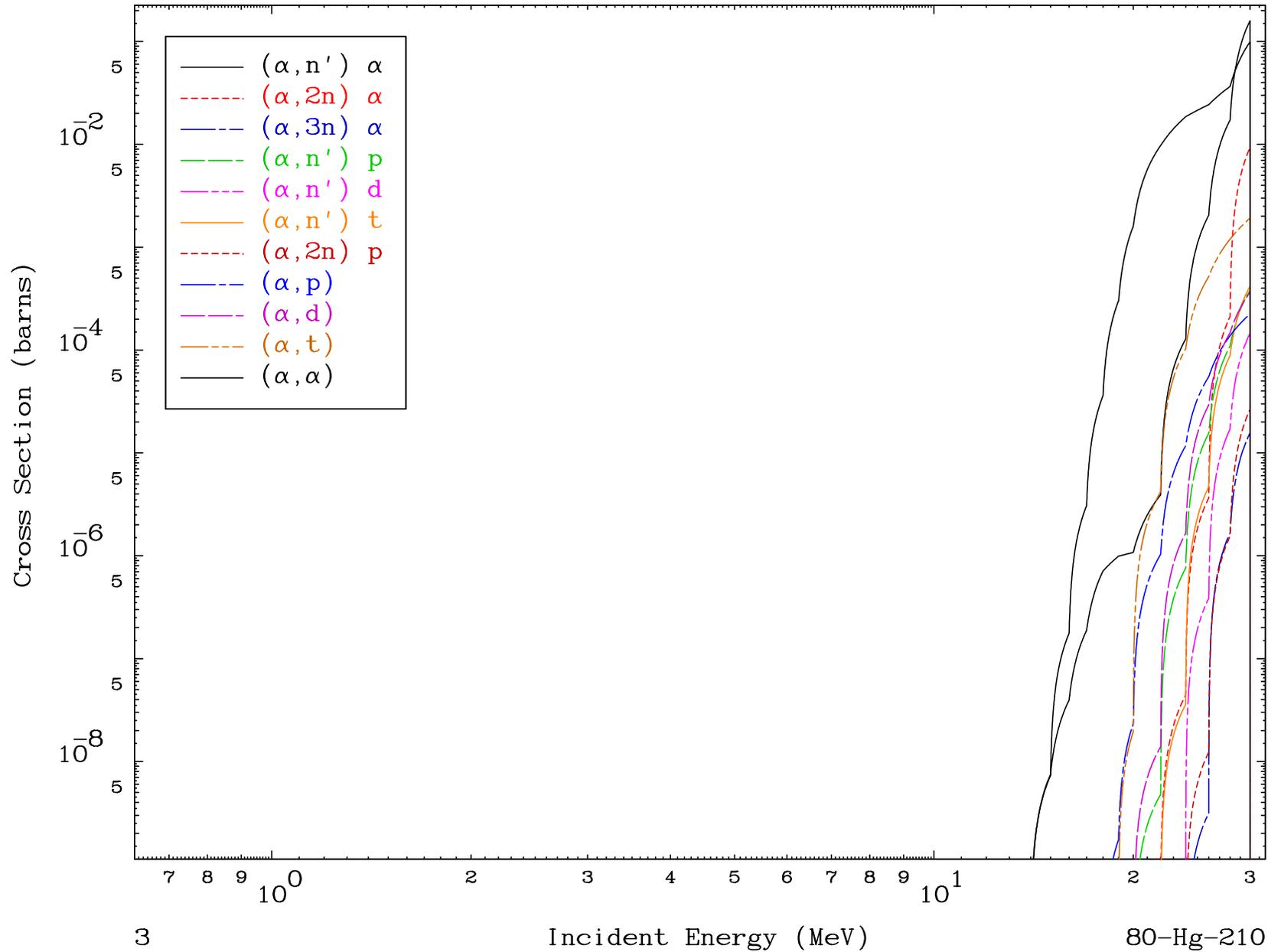


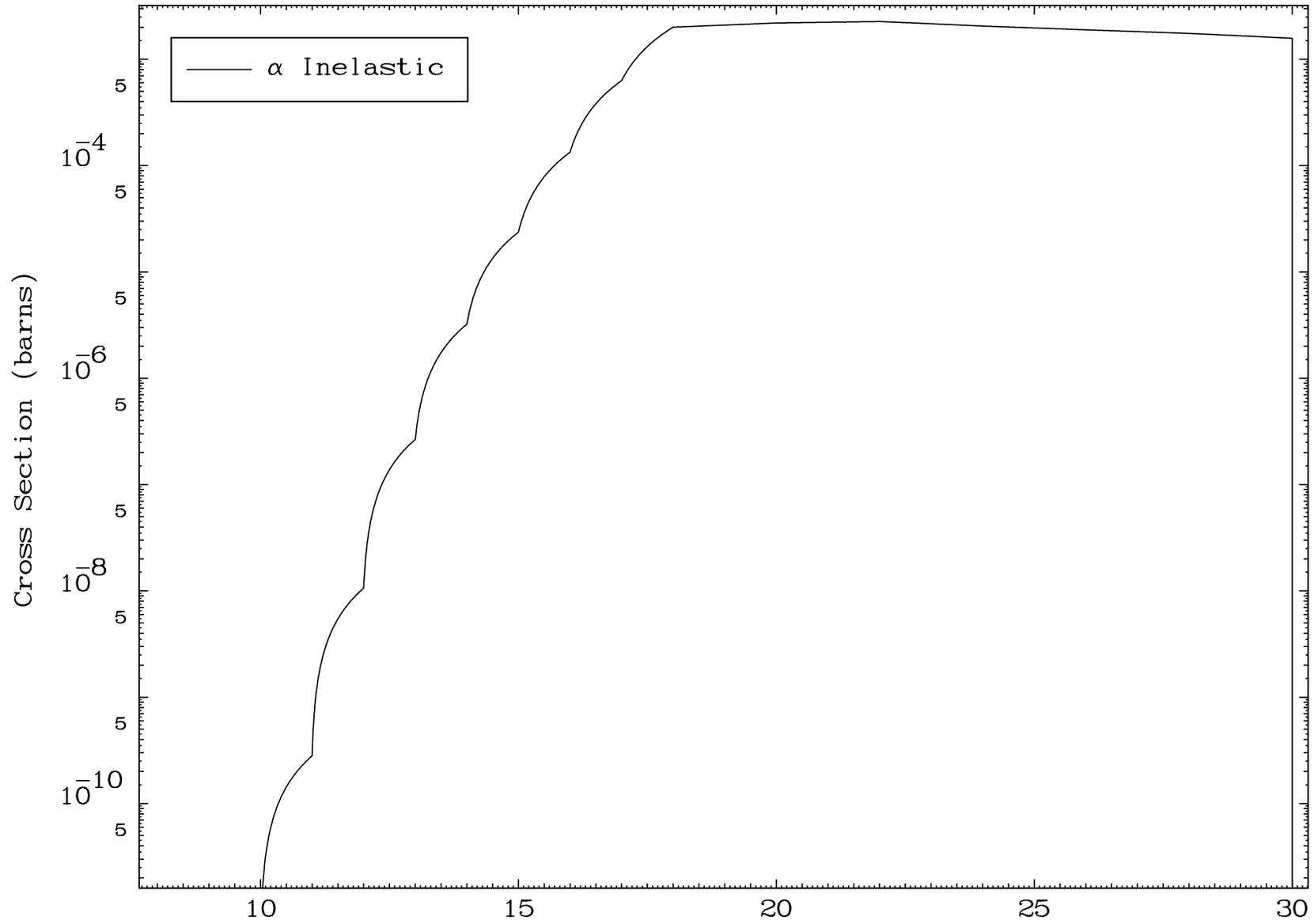
MAT 8067

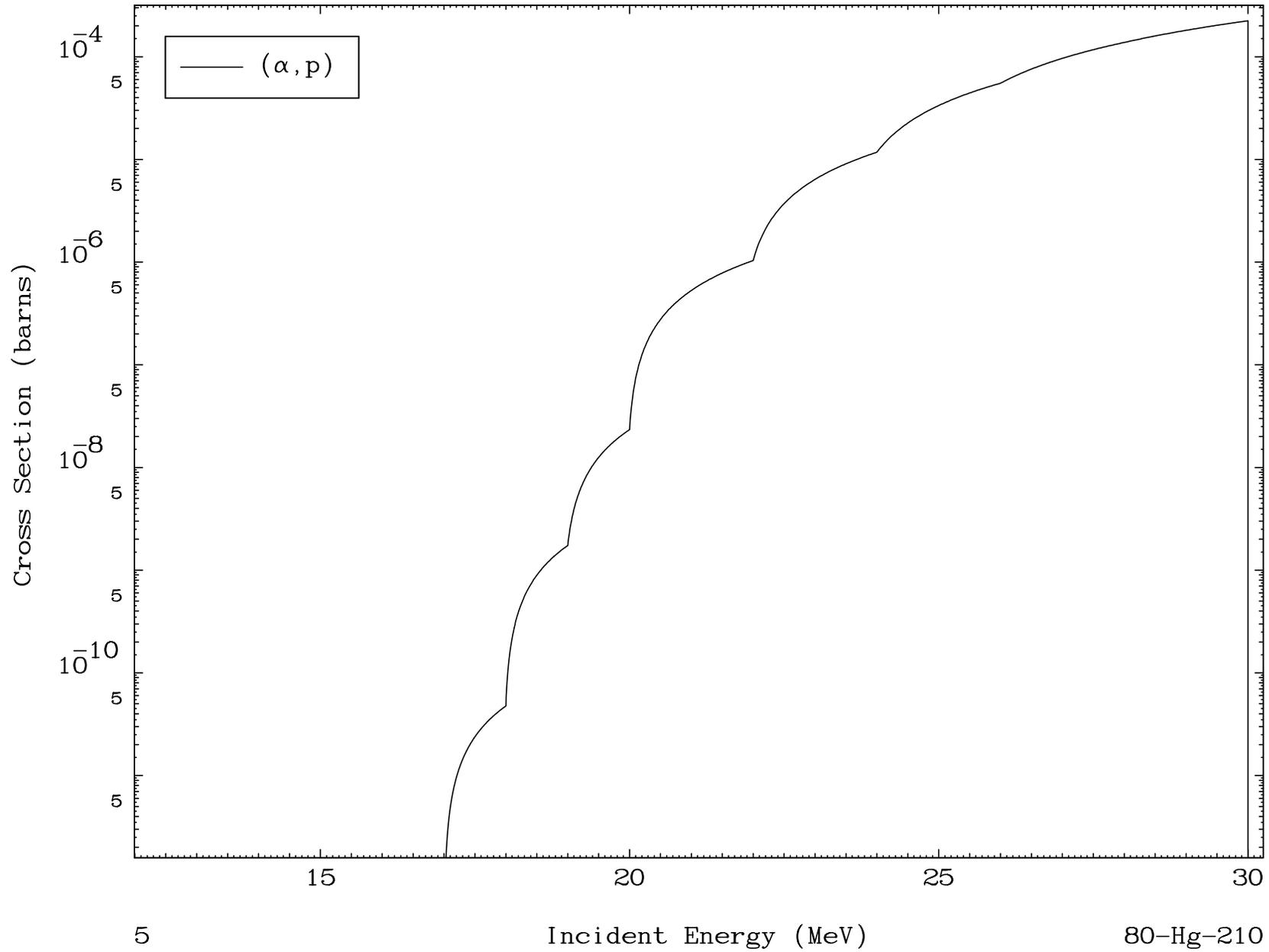
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

80-Hg-210





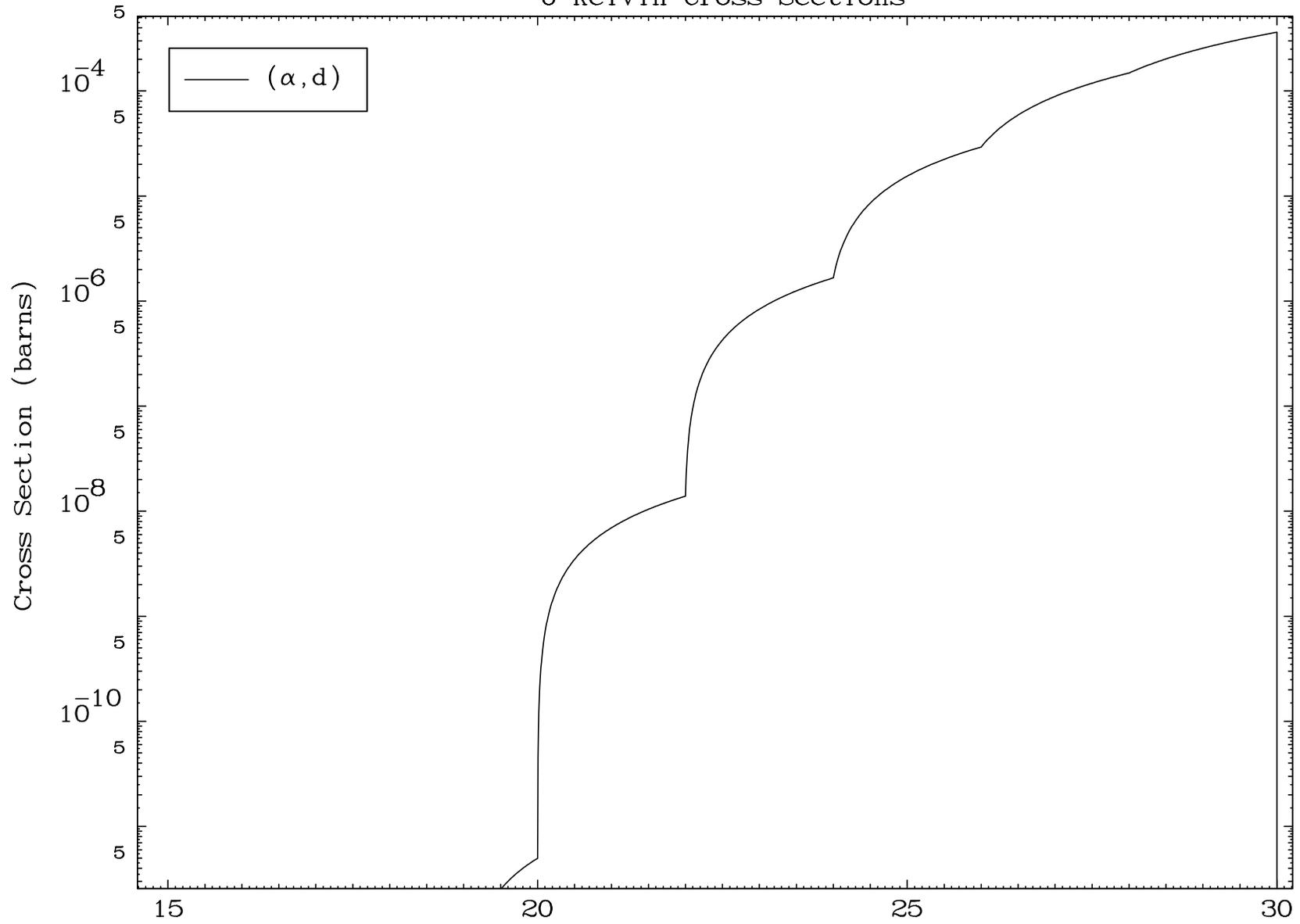




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( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

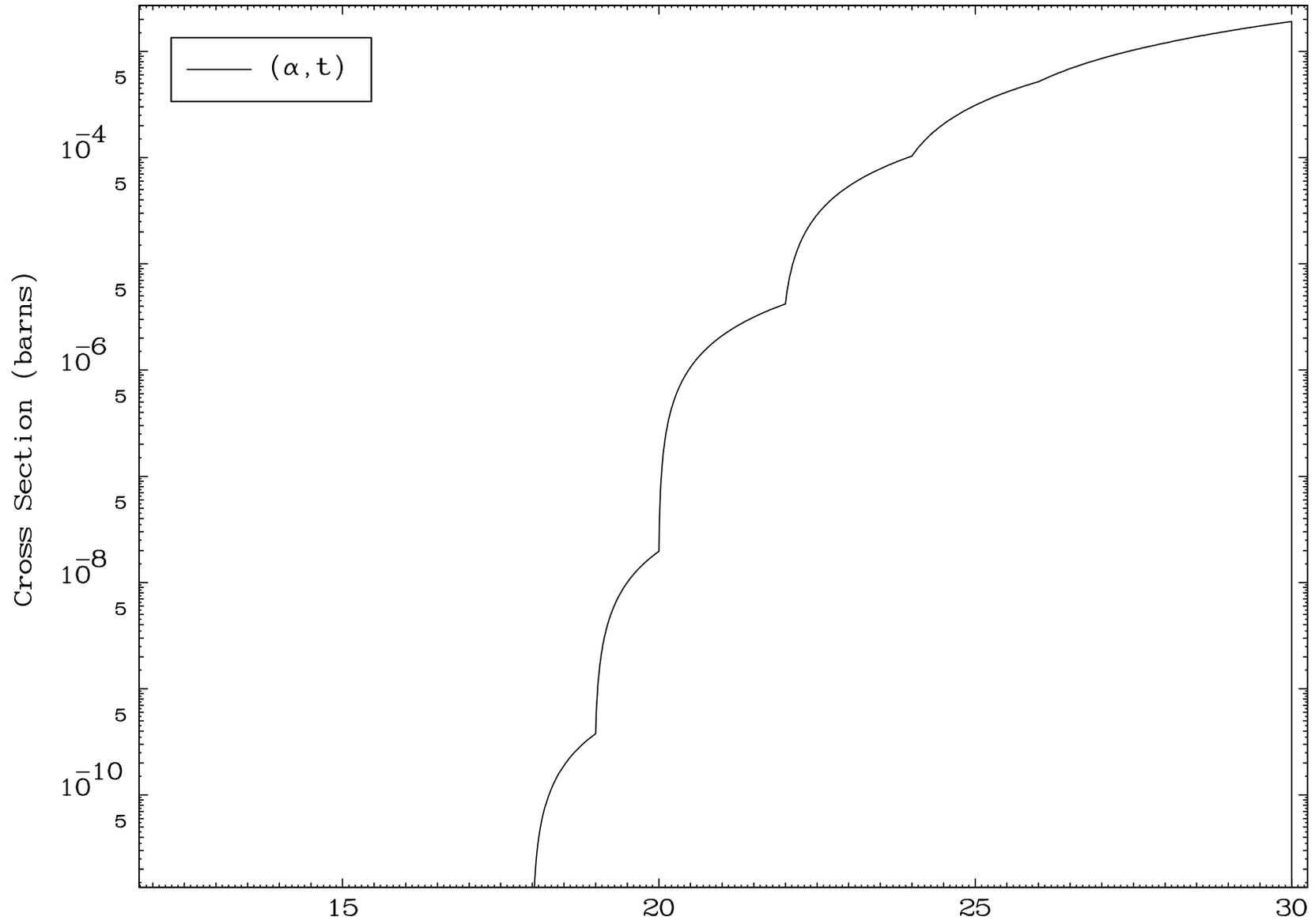
80-Hg-210



6

Incident Energy (MeV)

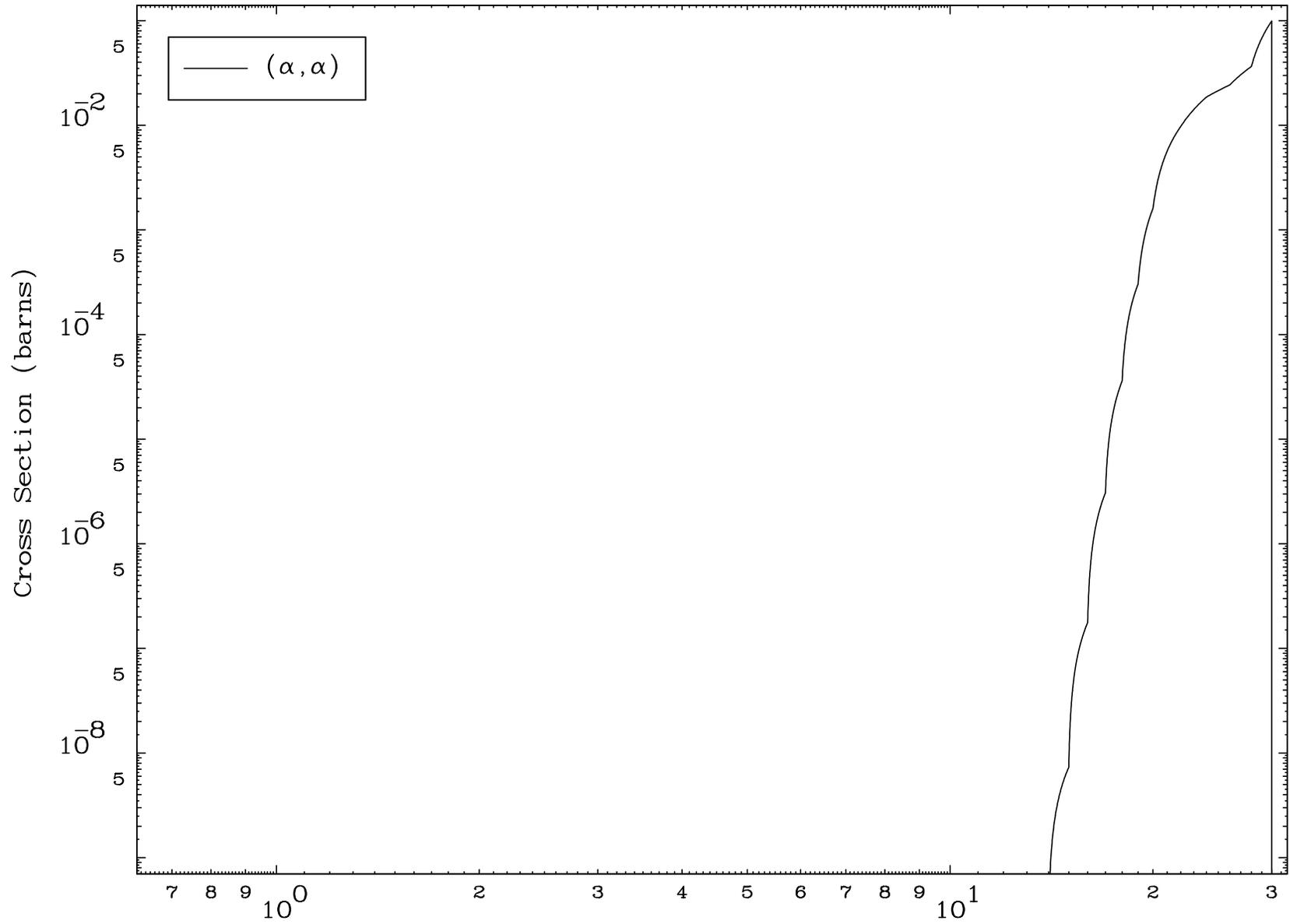
80-Hg-210



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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

80-Hg-210

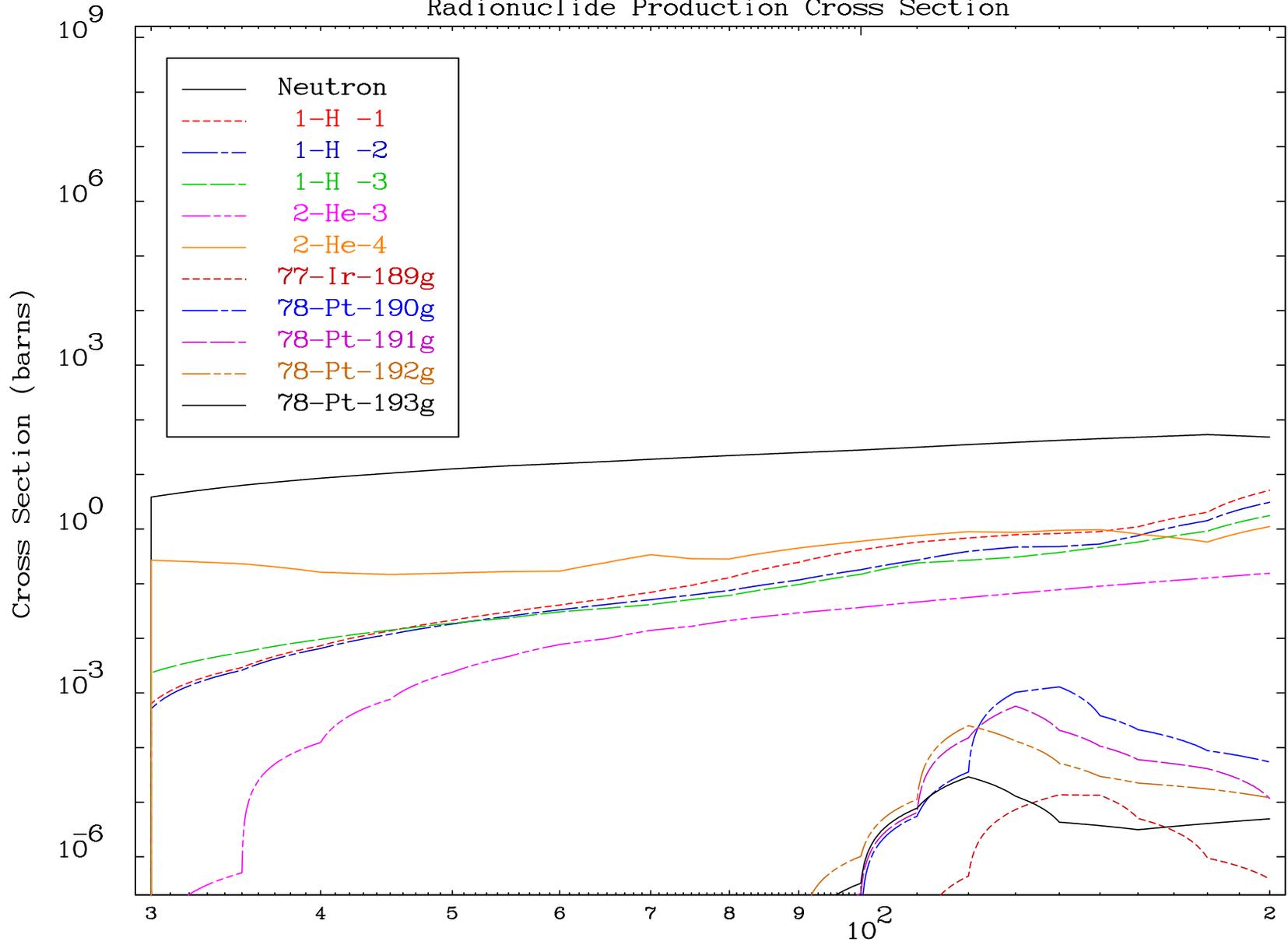


8

Incident Energy (MeV)

80-Hg-210

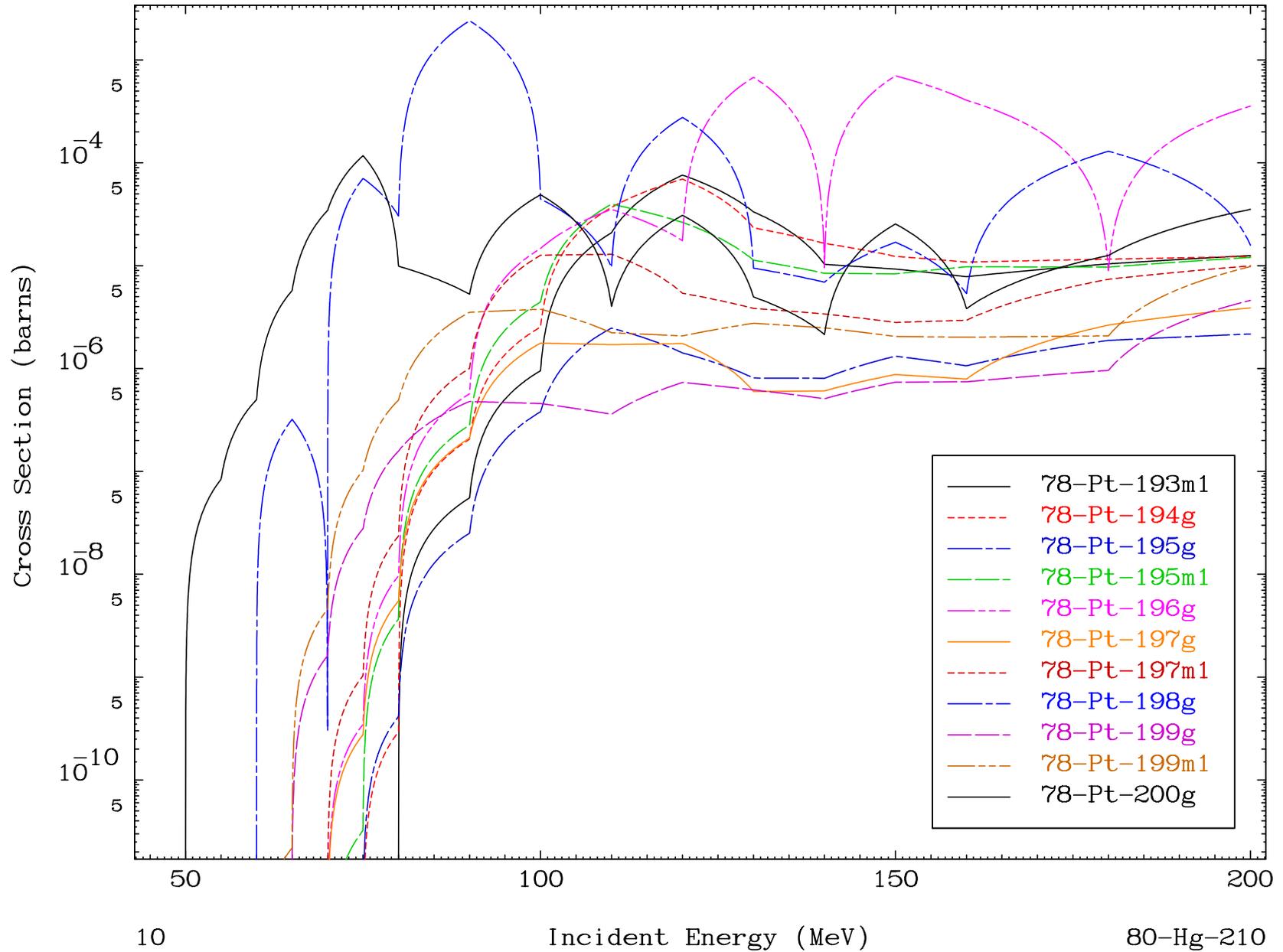
Radionuclide Production Cross Section



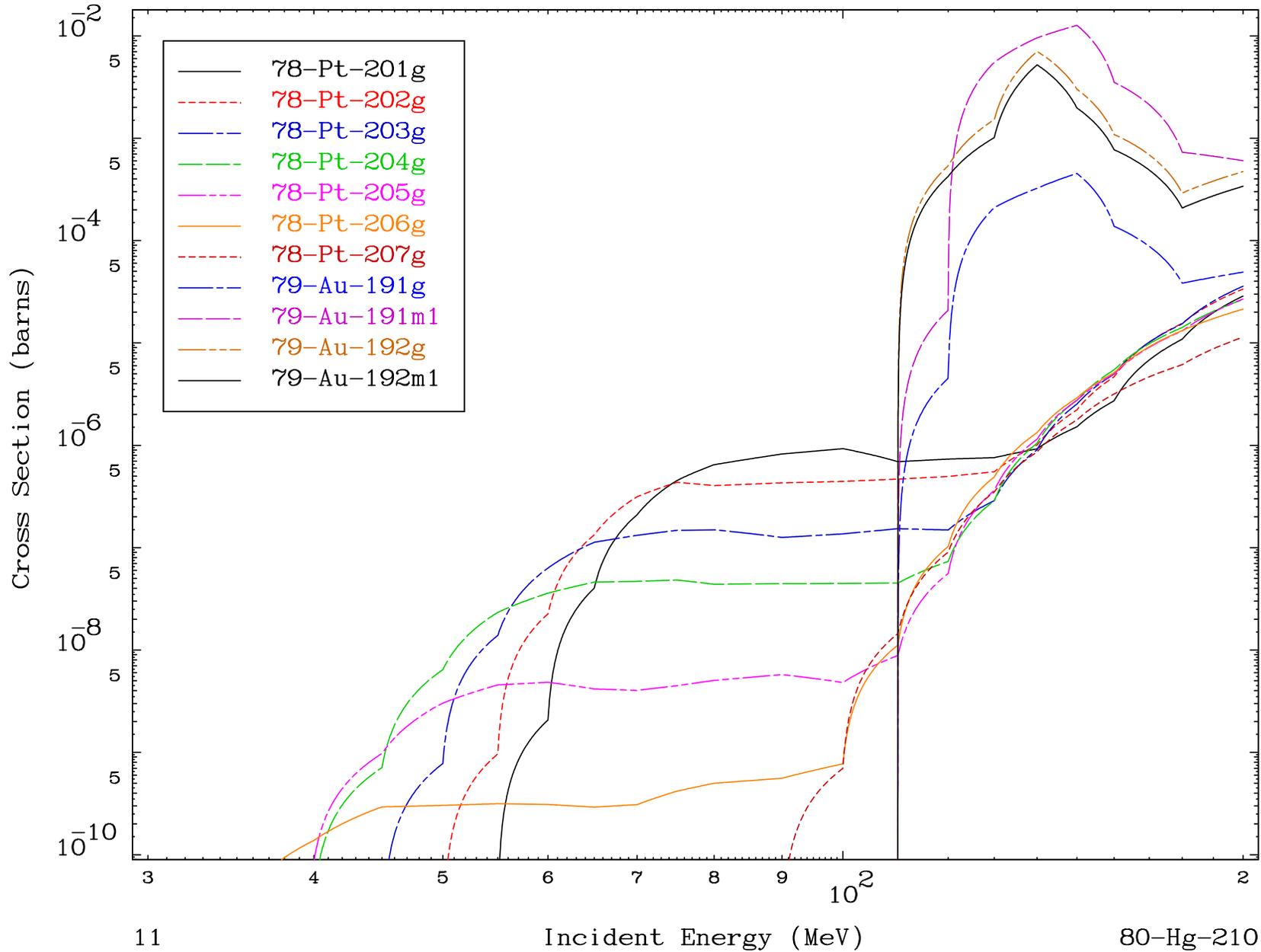
MAT 8067

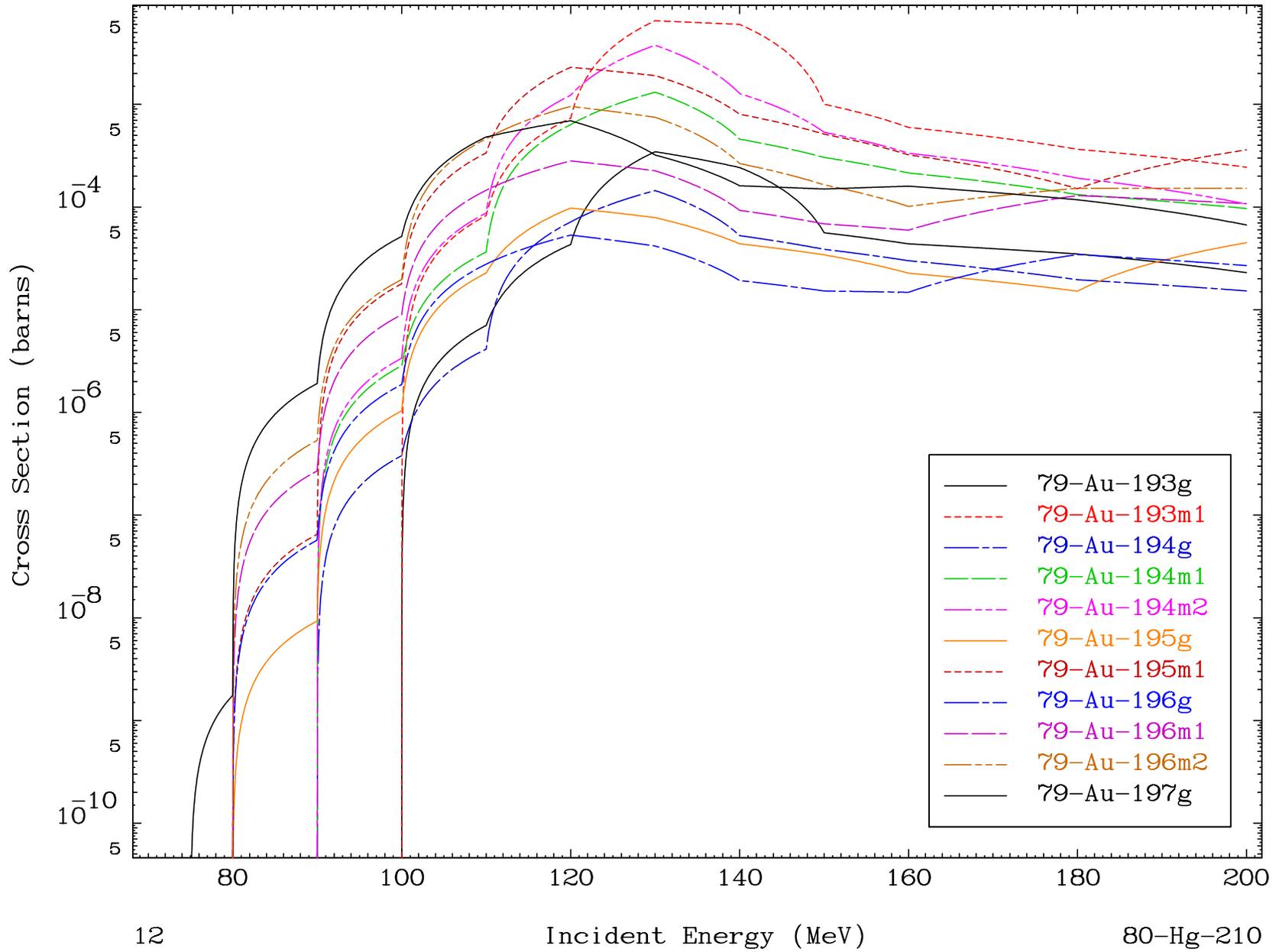
( $\alpha$ , remainder)  
Radionuclide Production Cross Section

80-Hg-210

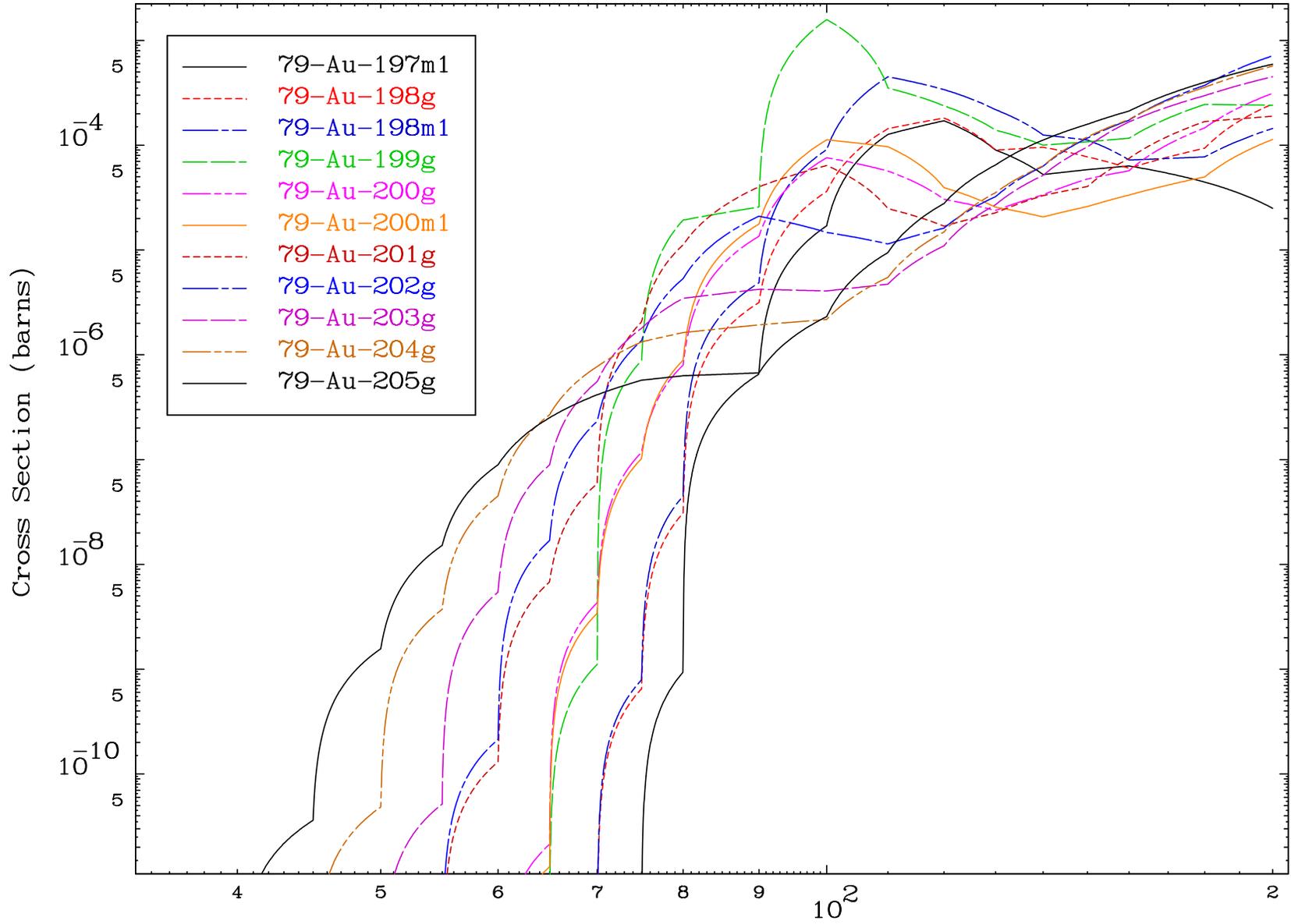


Radionuclide Production Cross Section

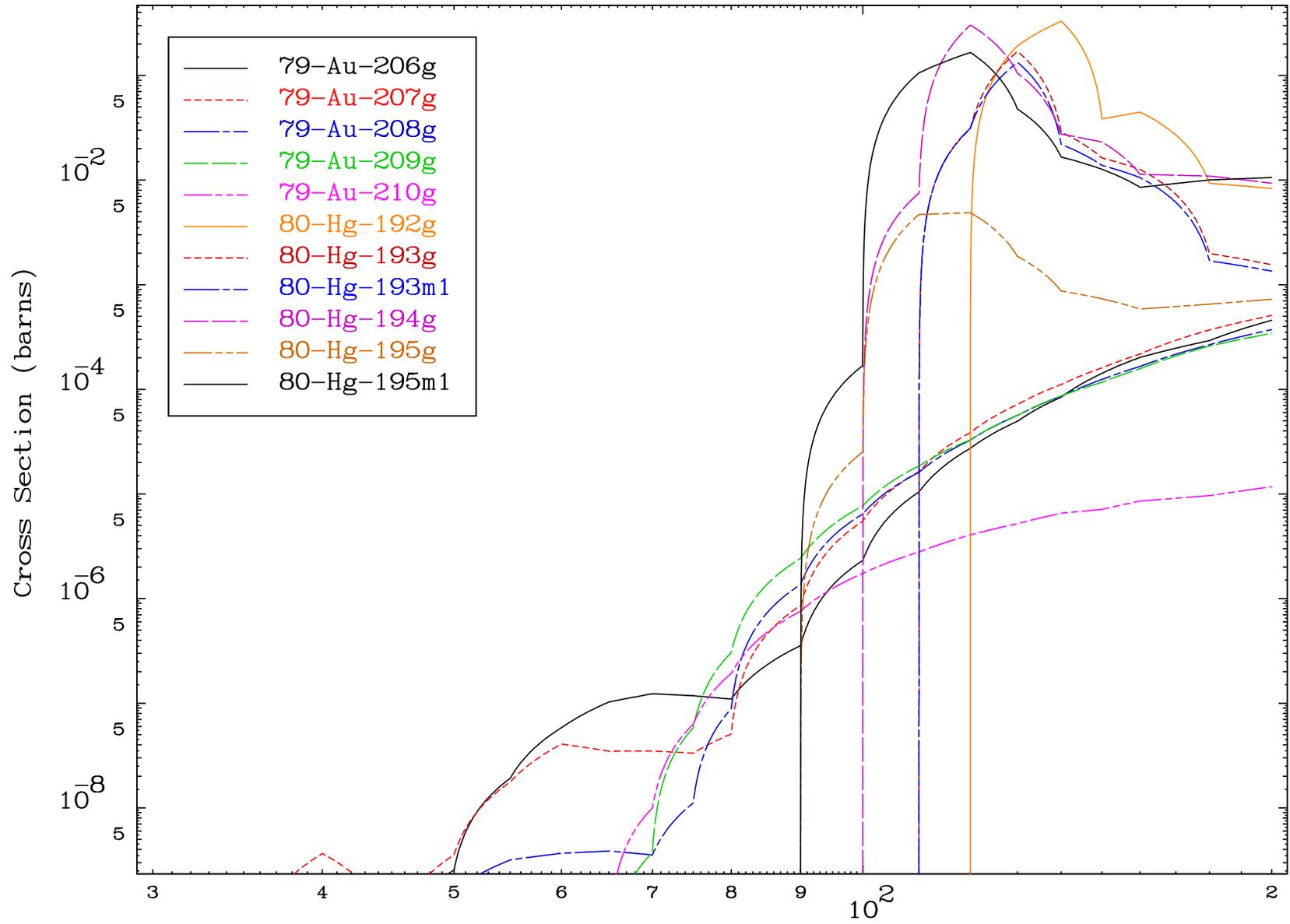




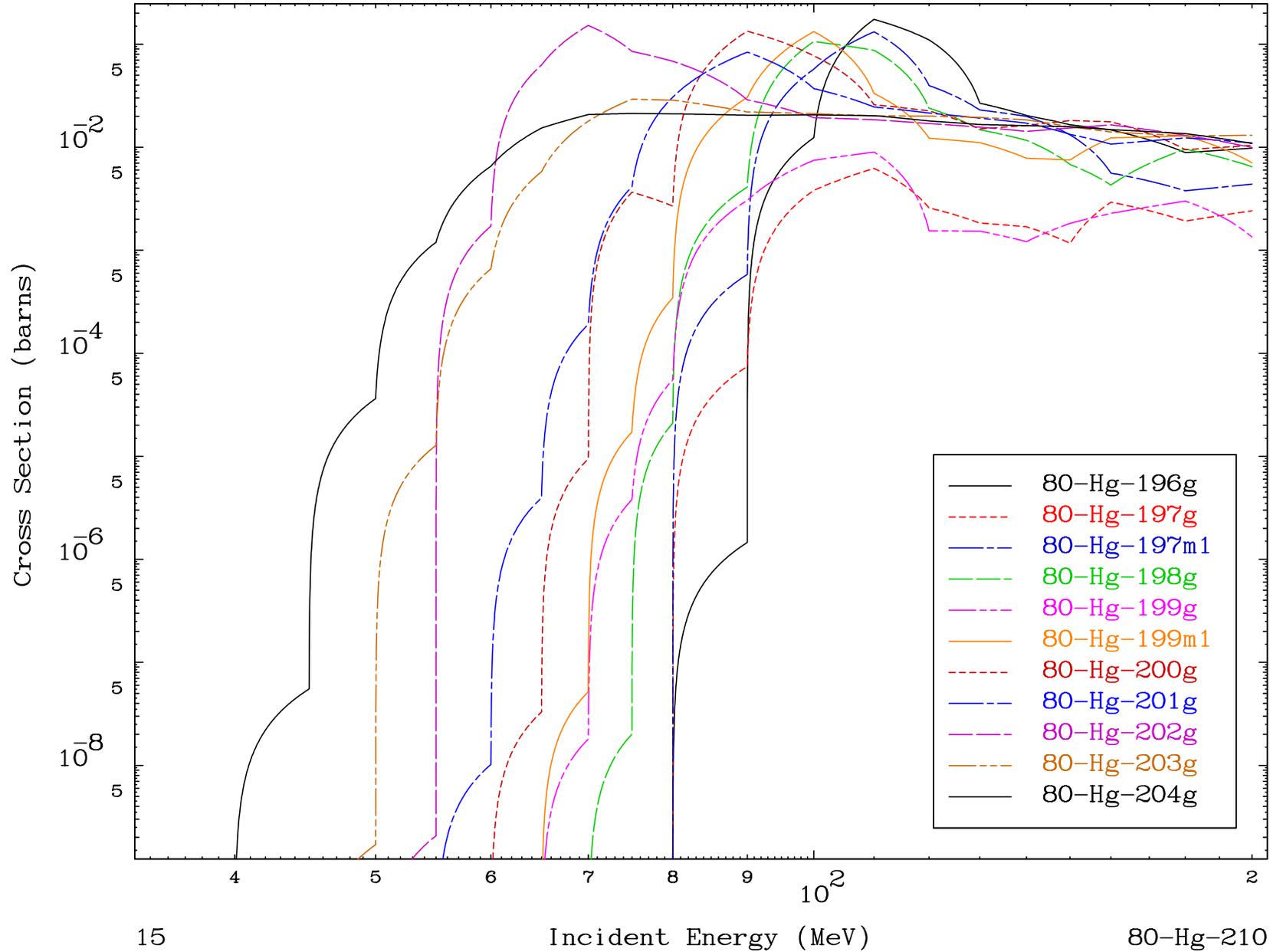
Radionuclide Production Cross Section



Radionuclide Production Cross Section



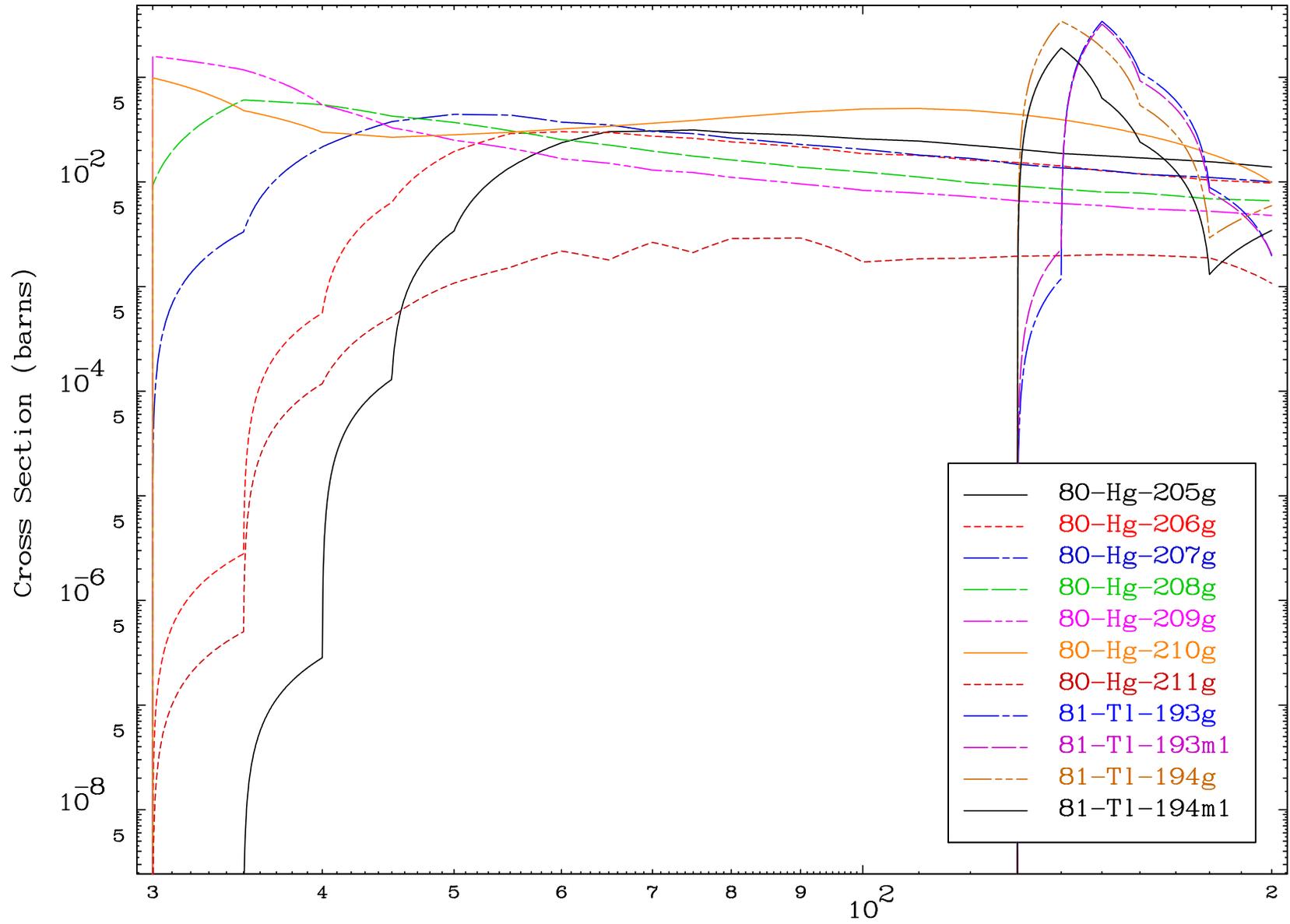
Radionuclide Production Cross Section



MAT 8067

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

80-Hg-210

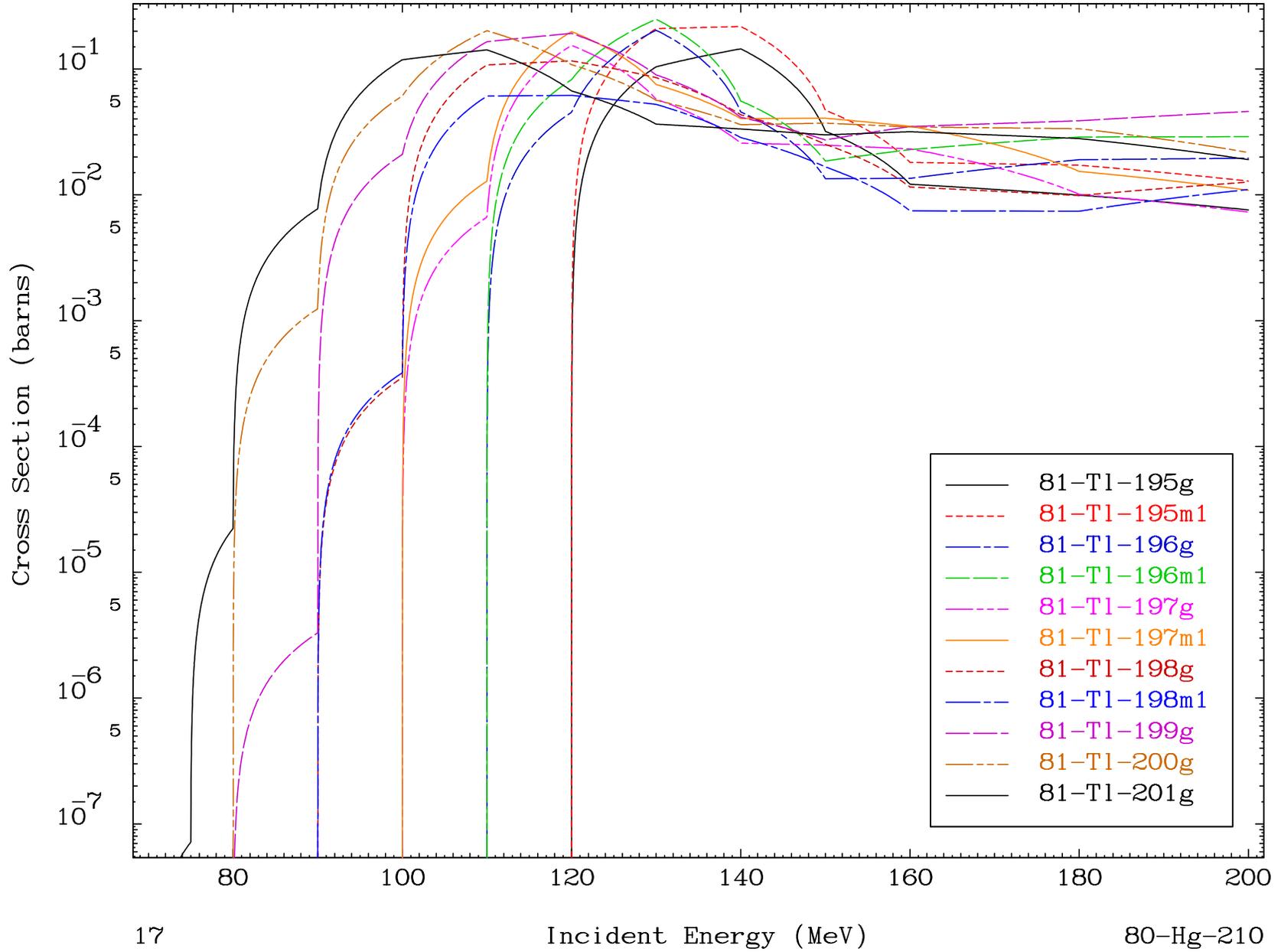


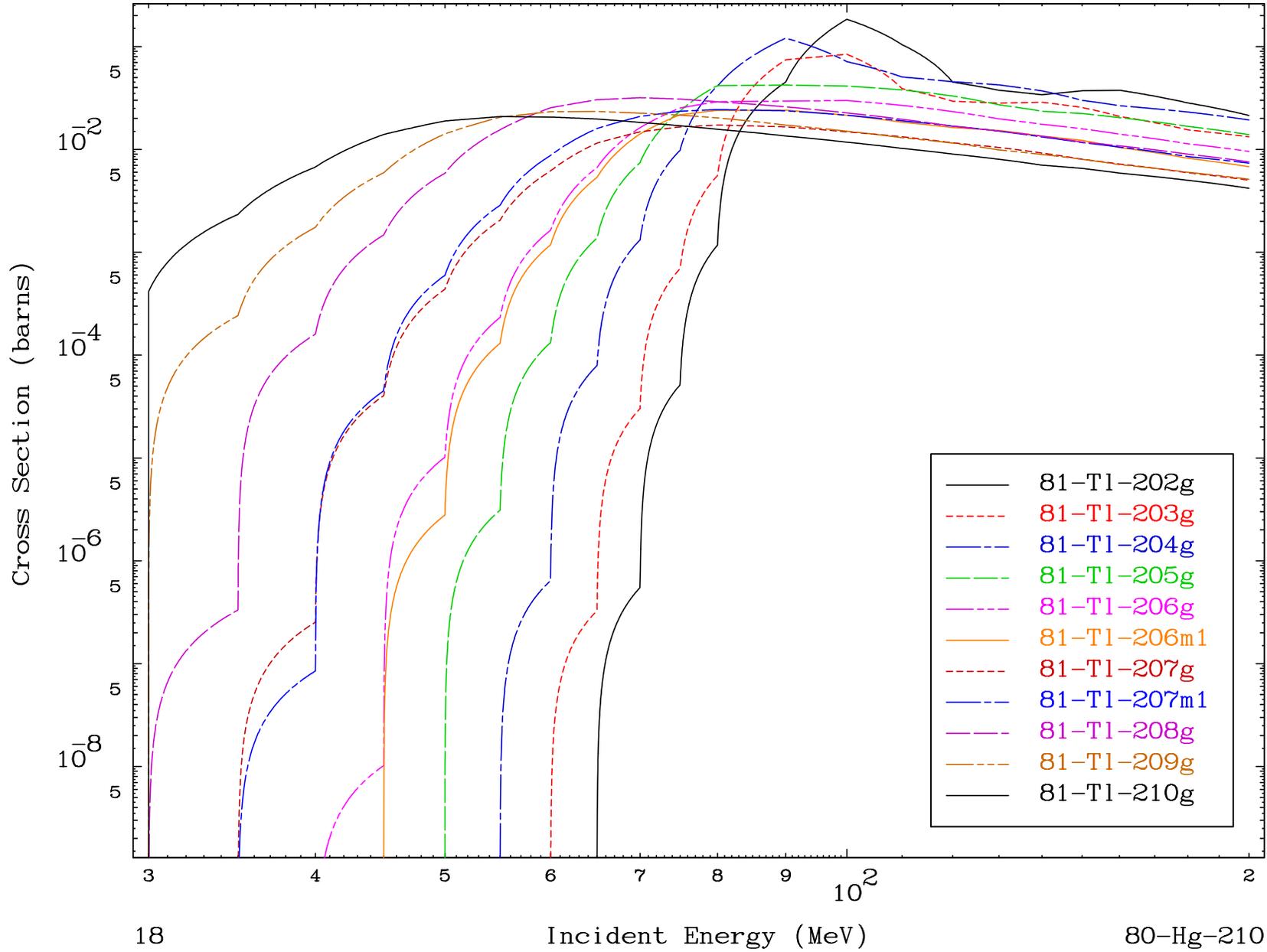
16

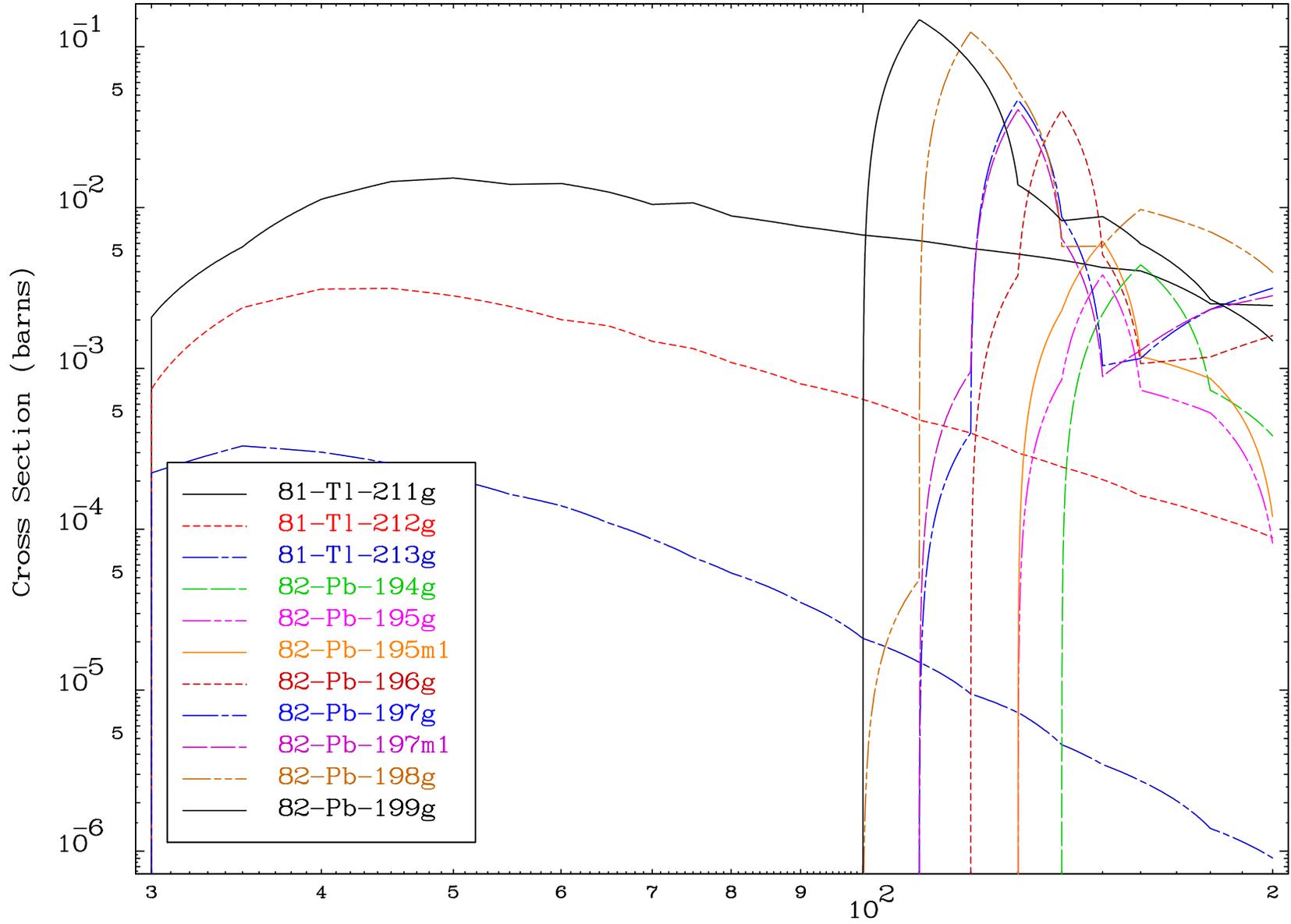
Incident Energy (MeV)

80-Hg-210

Radionuclide Production Cross Section





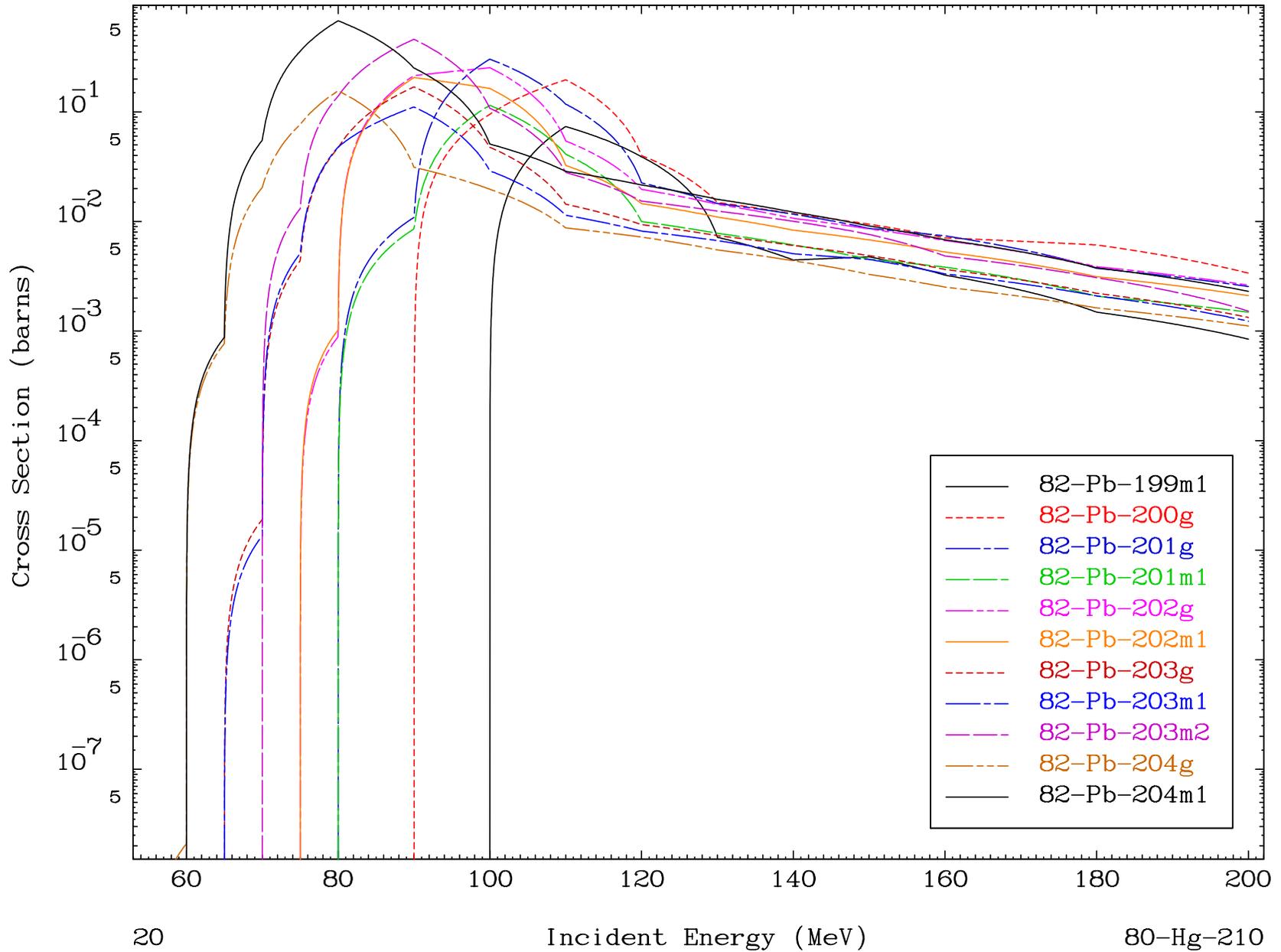


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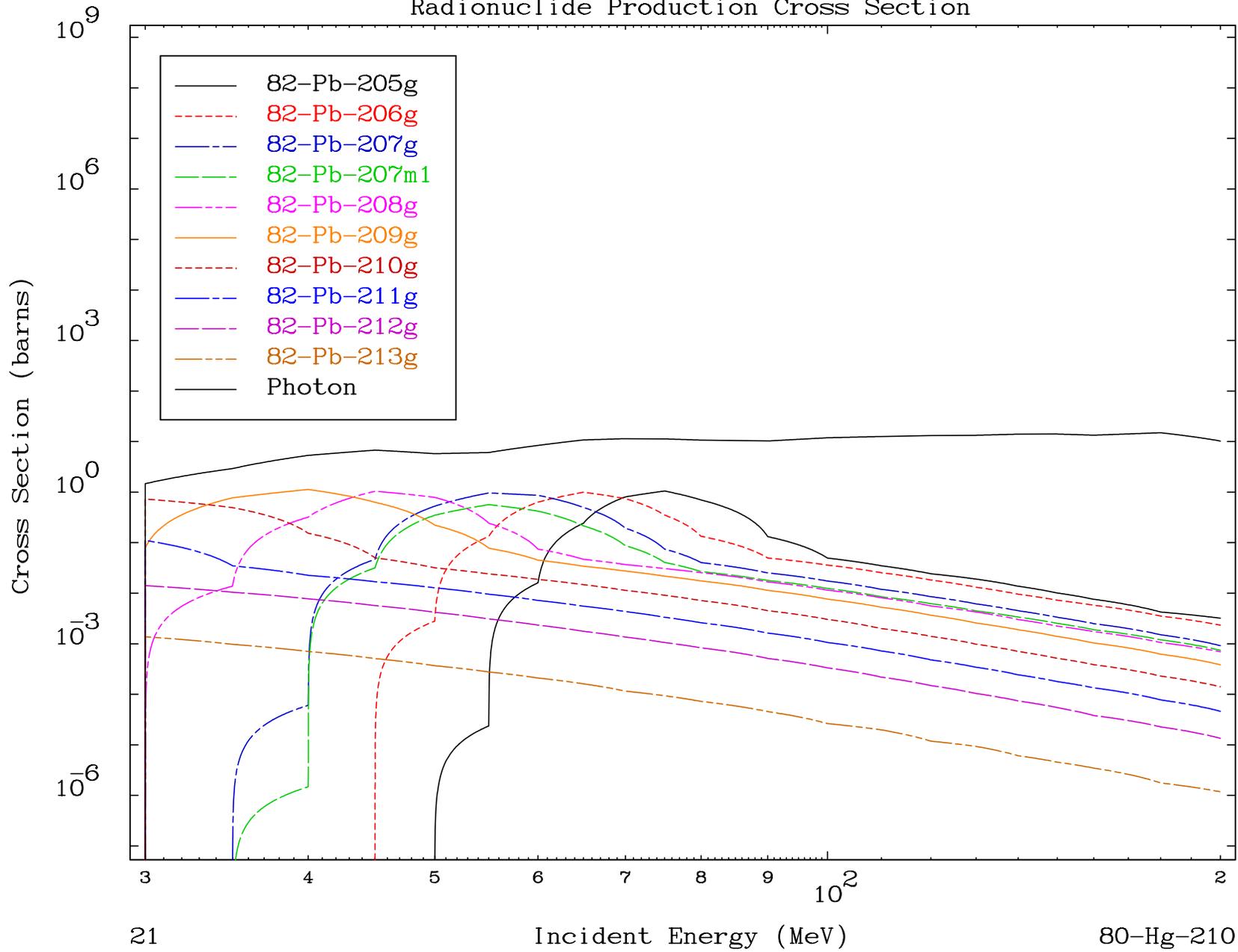
( $\alpha$ , remainder)

80-Hg-210

### Radionuclide Production Cross Section



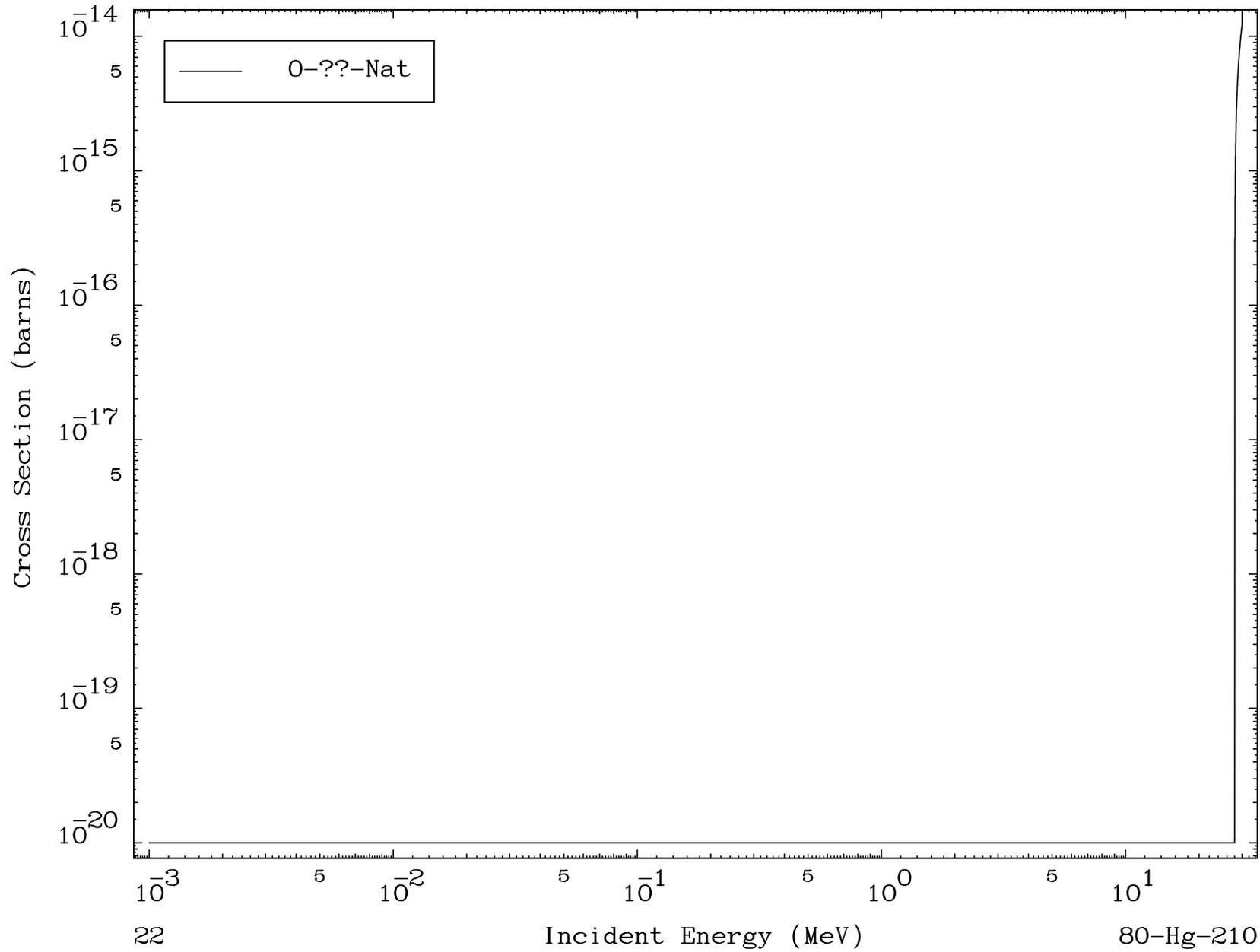
Radionuclide Production Cross Section



MAT 8067

$\alpha$  Fission  
Radionuclide Production Cross Section

80-Hg-210



22

Incident Energy (MeV)

80-Hg-210