

Program EVALPLOT  
(Version 2018-1)

by

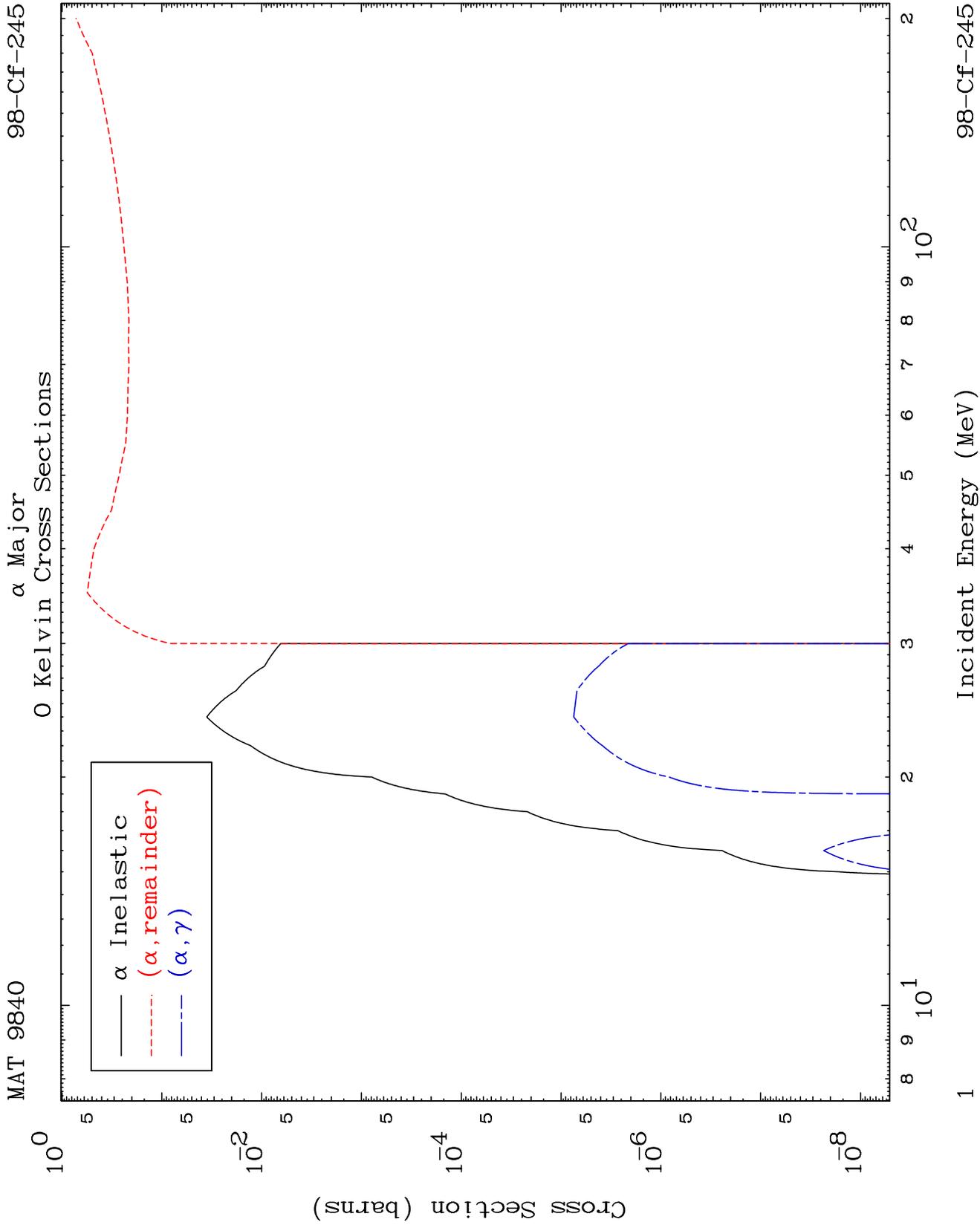
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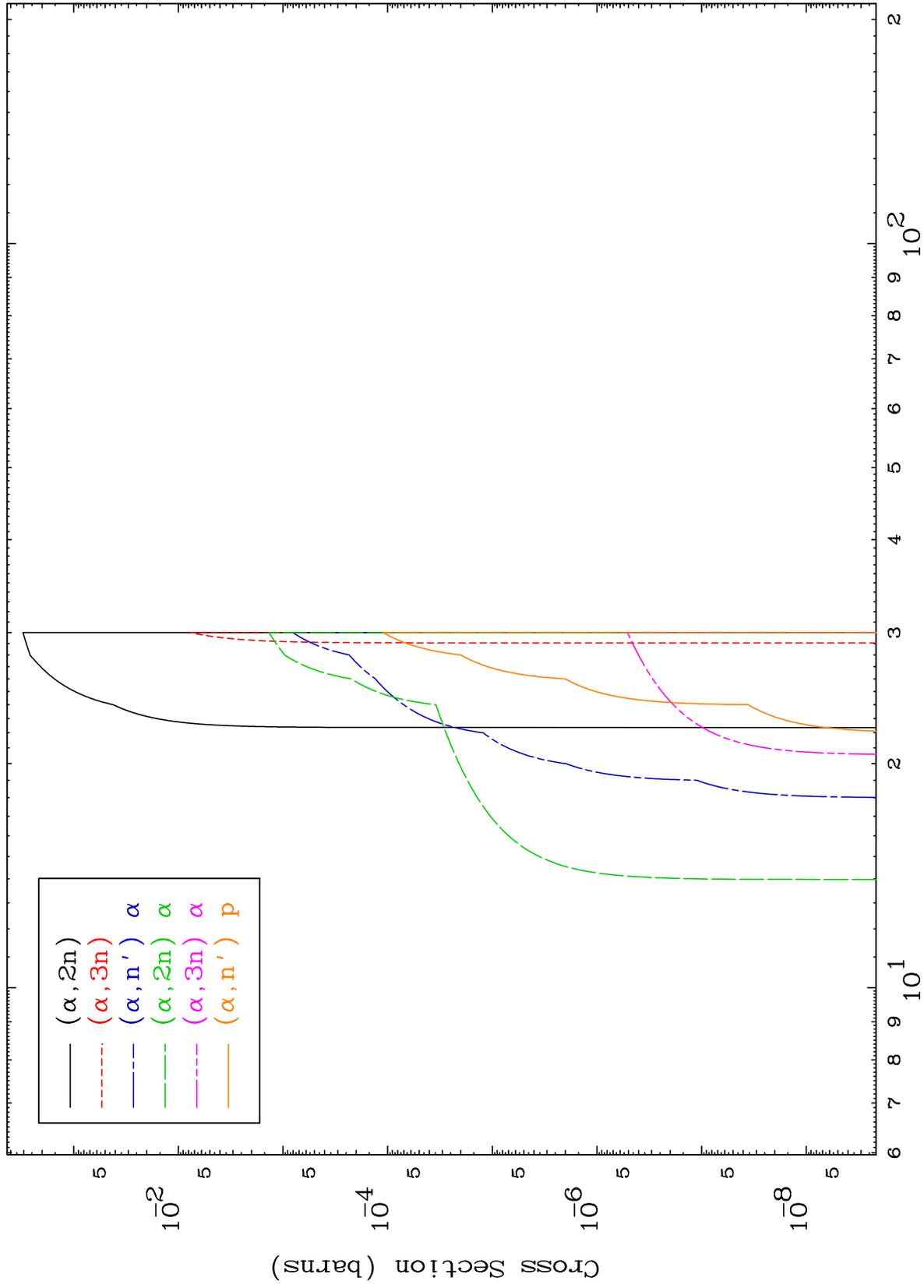
Press Mouse Button to Start



MAT 9840

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

98-Cf-245



2

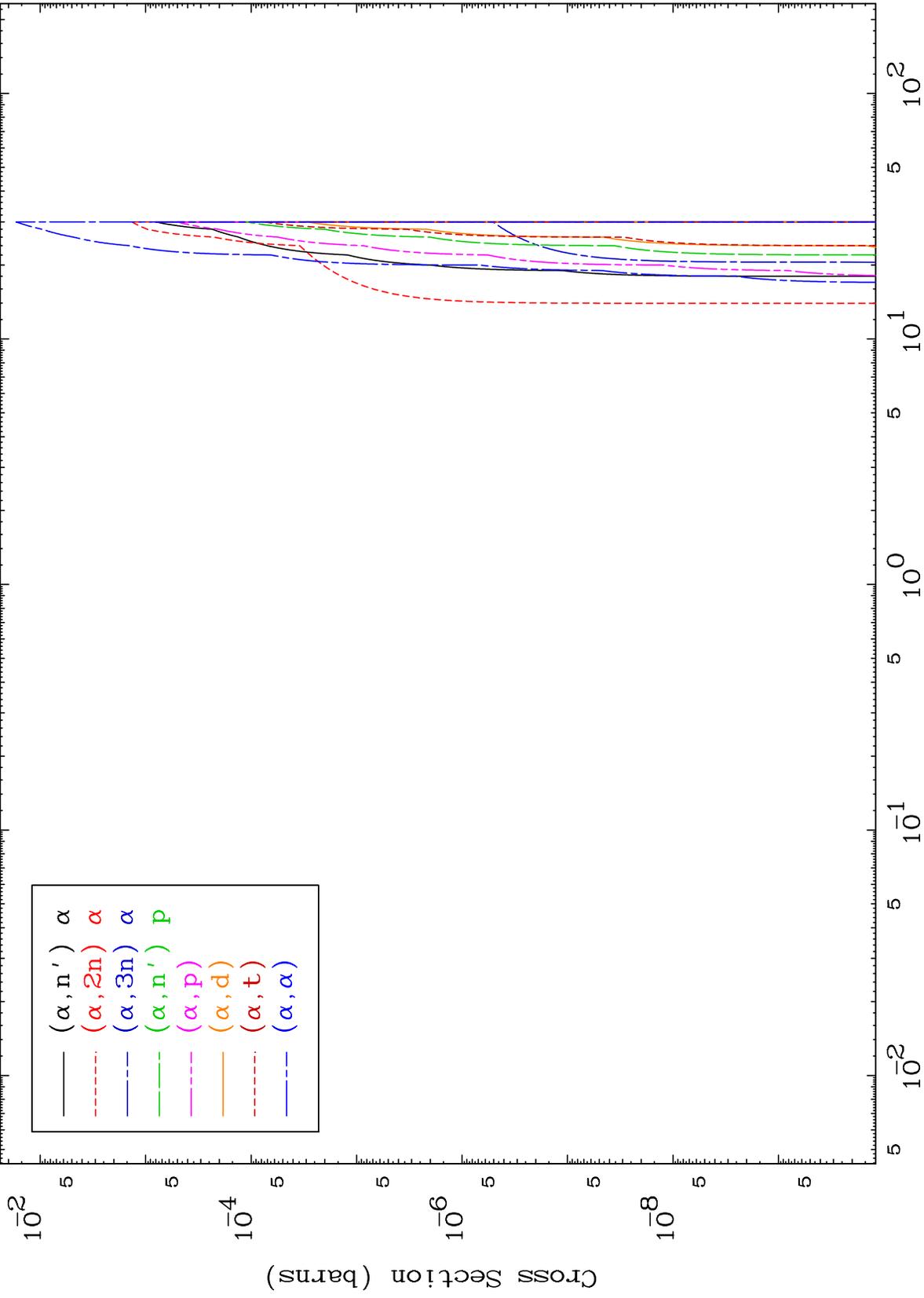
Incident Energy (MeV)

98-Cf-245

MAT 9840

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

98-Cf-245



3

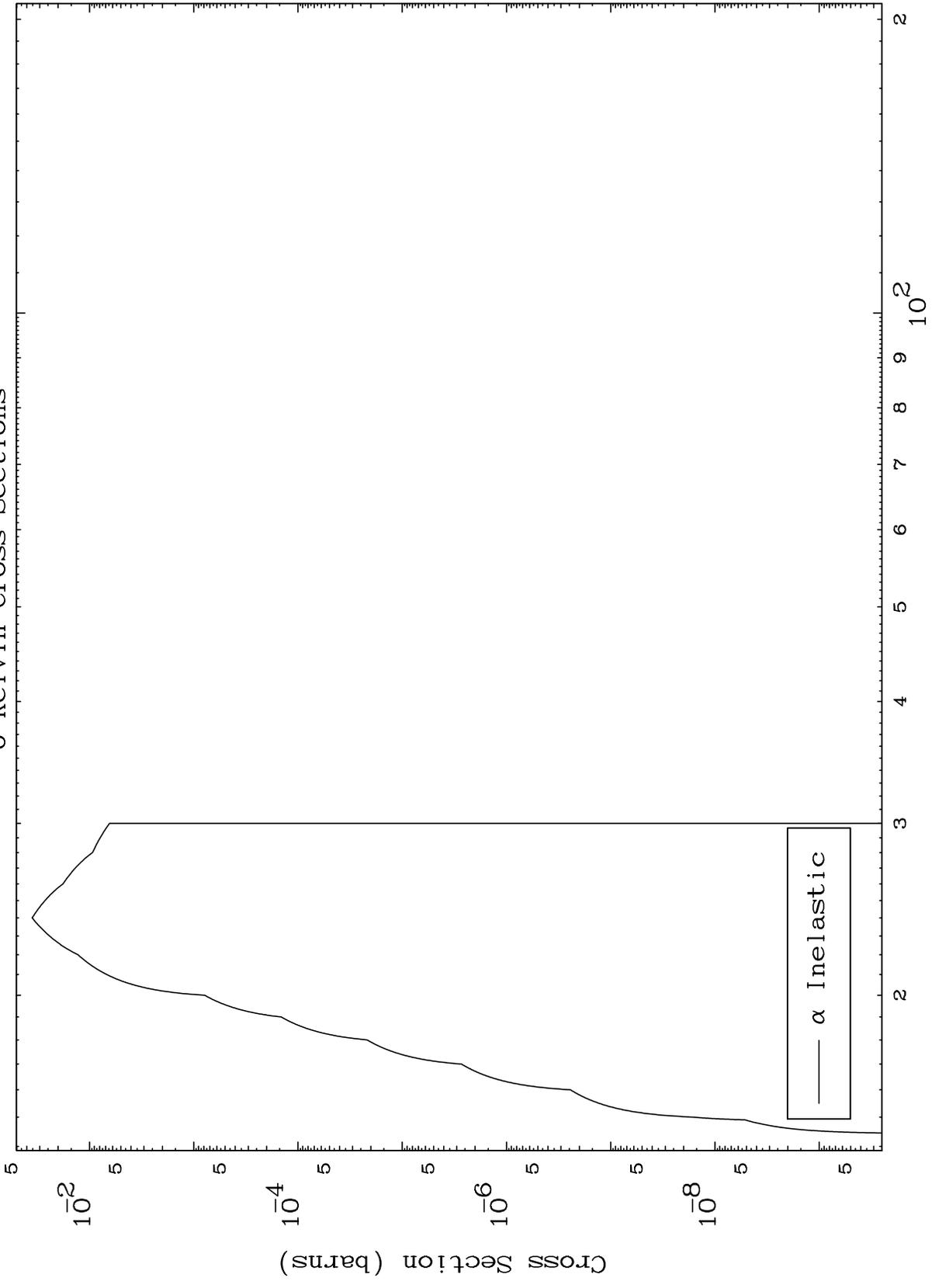
Incident Energy (MeV)

98-Cf-245

MAT 9840

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

98-Cf-245



4

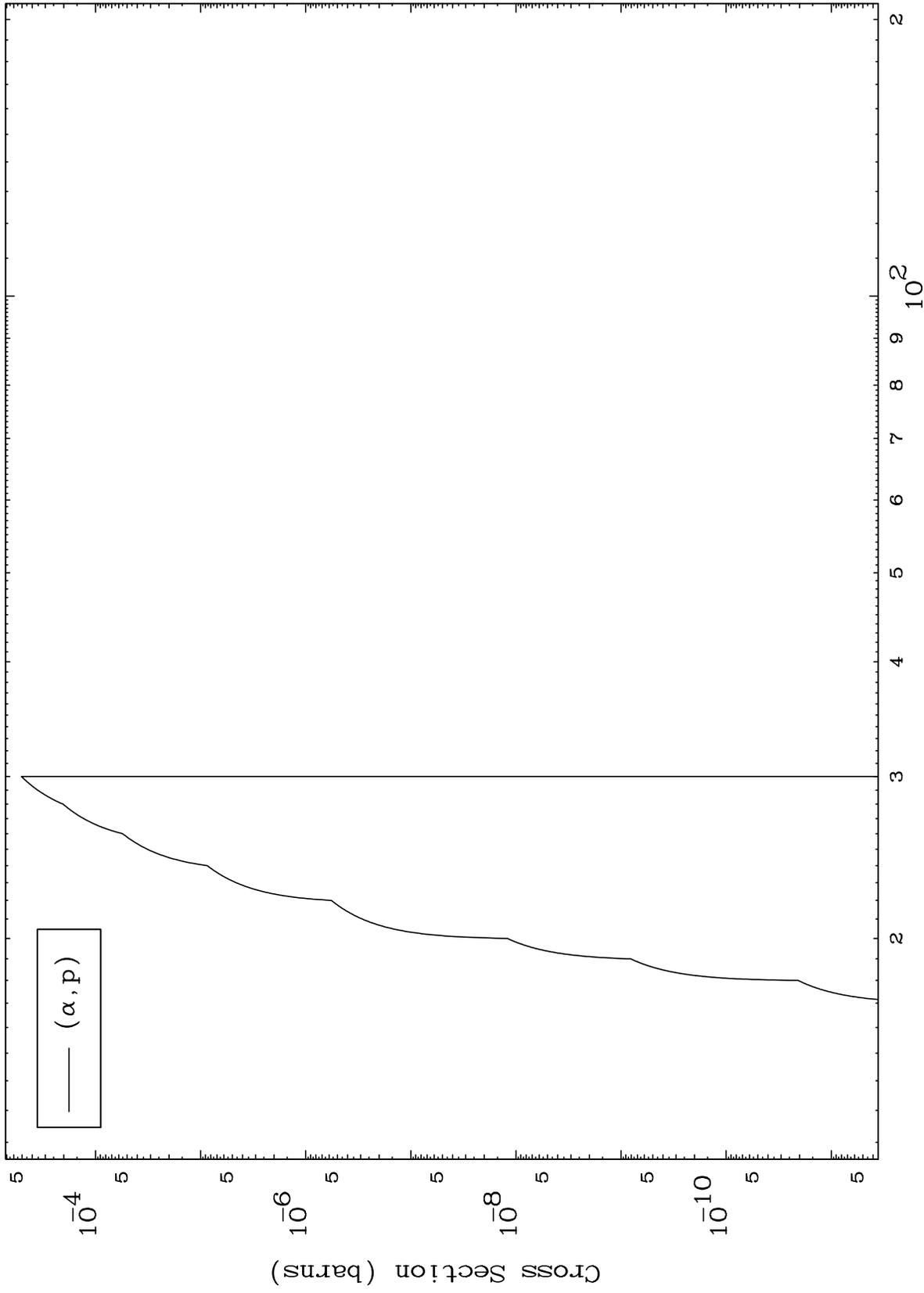
Incident Energy (MeV)

98-Cf-245

MAT 9840

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

98-Cf-245



5

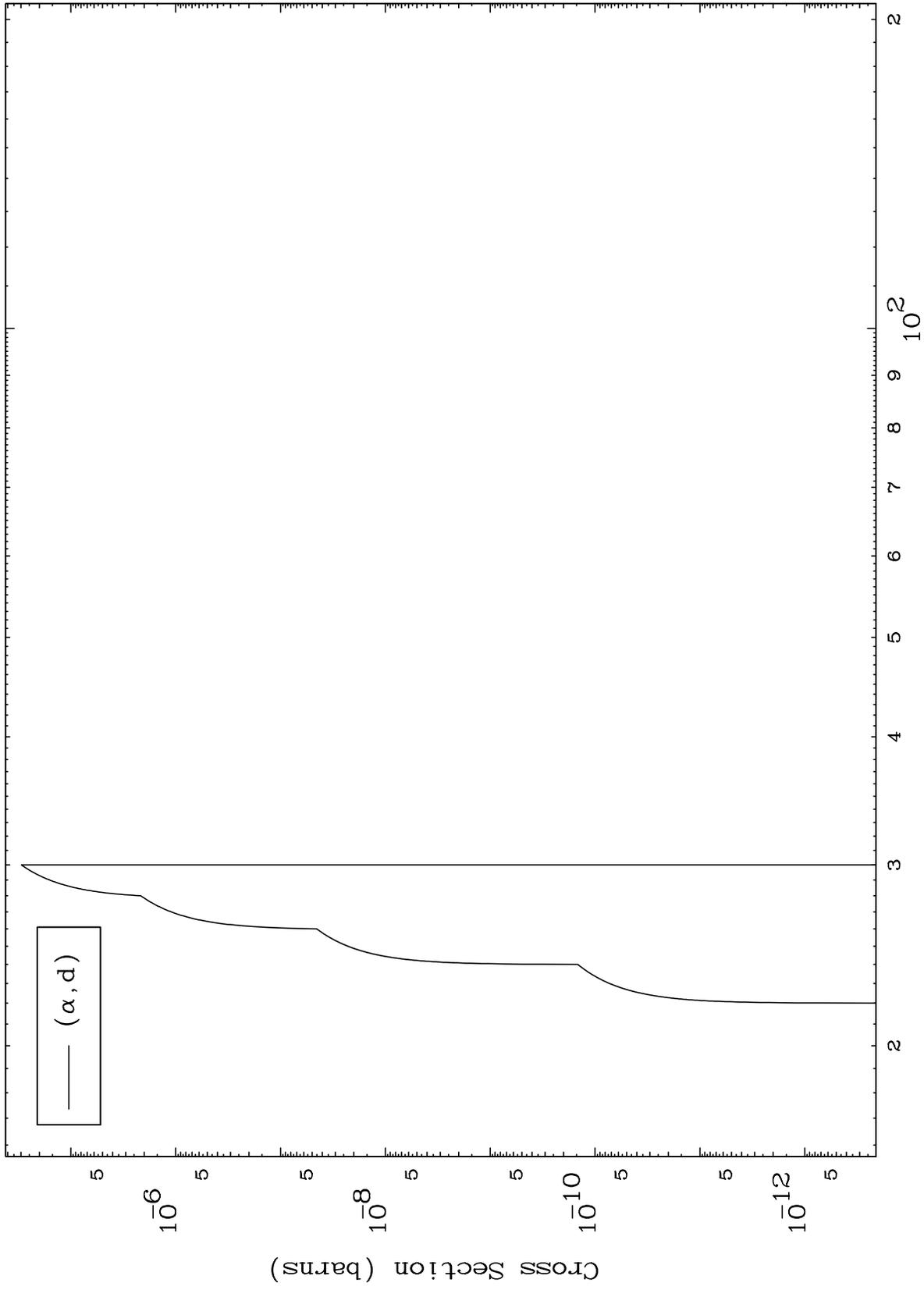
Incident Energy (MeV)

98-Cf-245

MAT 9840

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

98-Cf-245



6

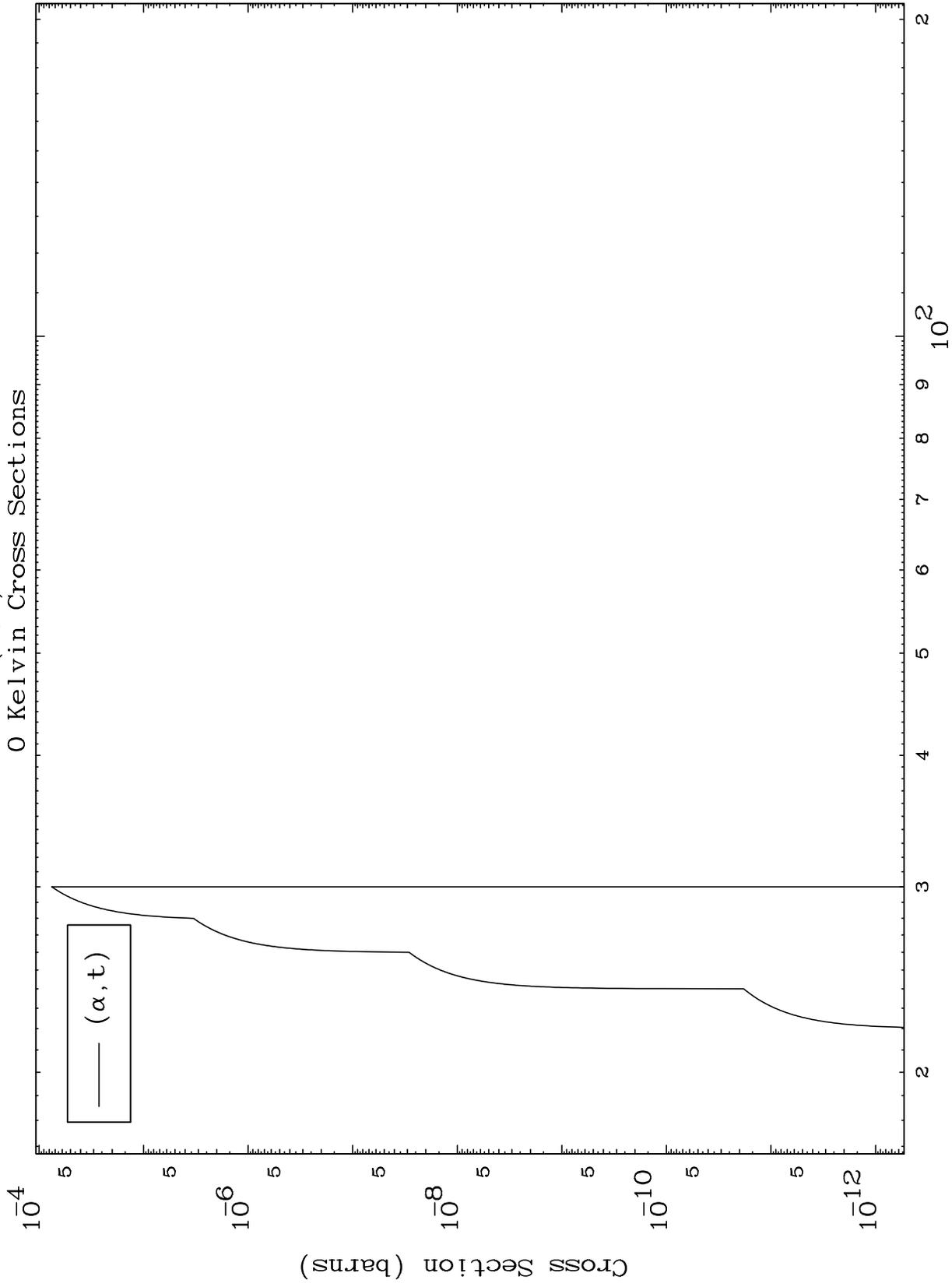
Incident Energy (MeV)

98-Cf-245

MAT 9840

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

98-Cf-245



7

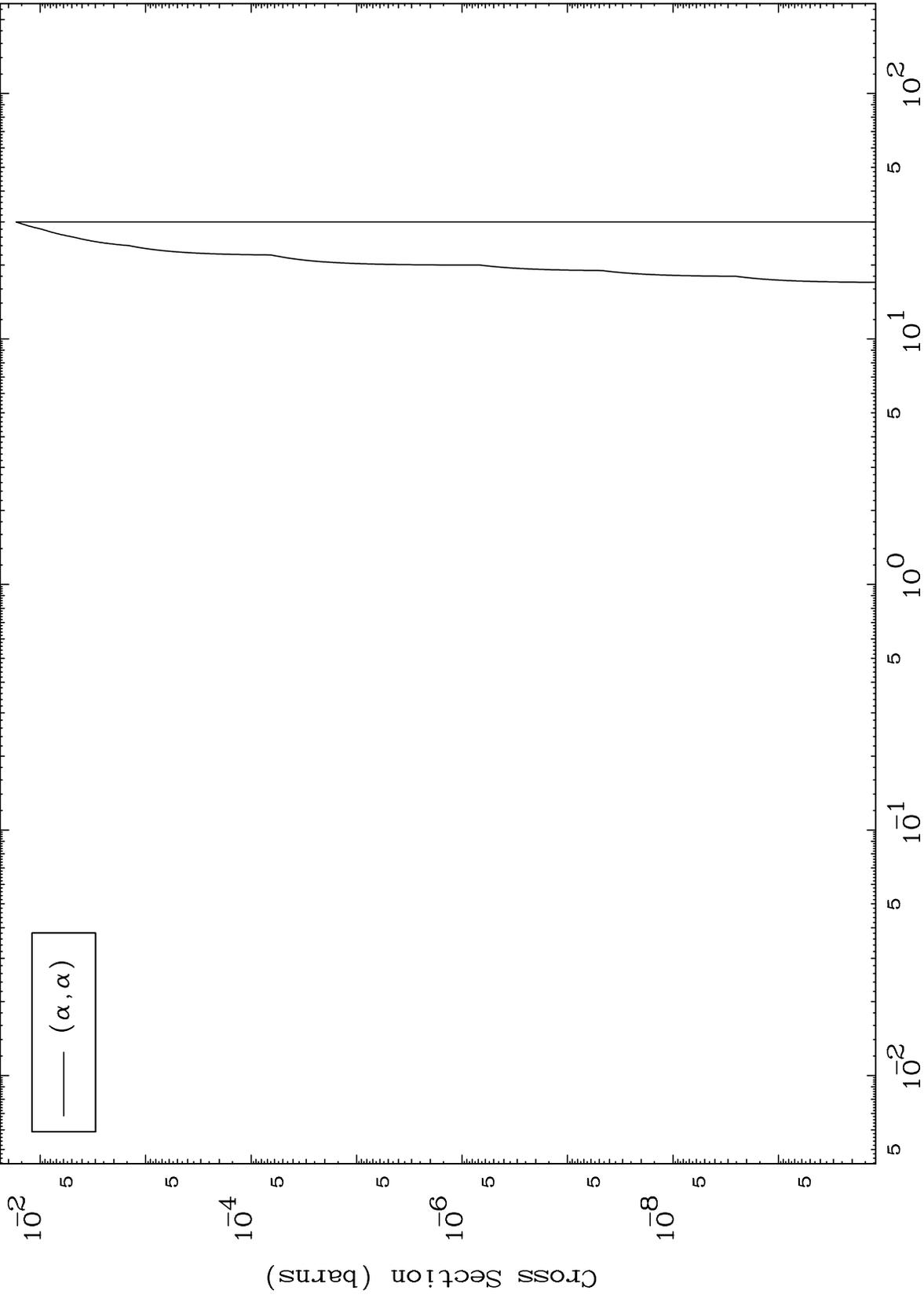
Incident Energy (MeV)

98-Cf-245

MAT 9840

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

98-Cf-245



8

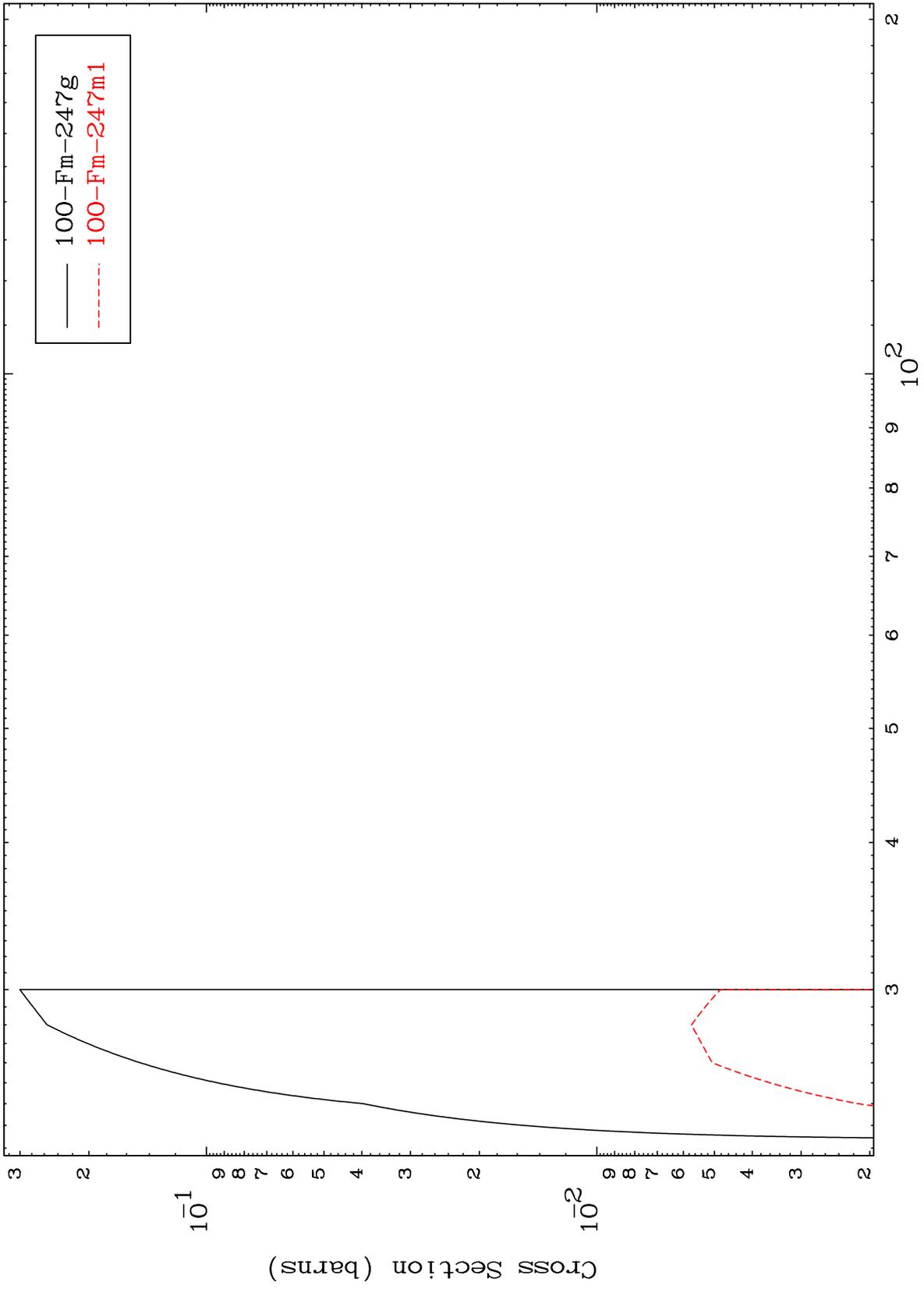
Incident Energy (MeV)

98-Cf-245

MAT 9840

98-Cf-245

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



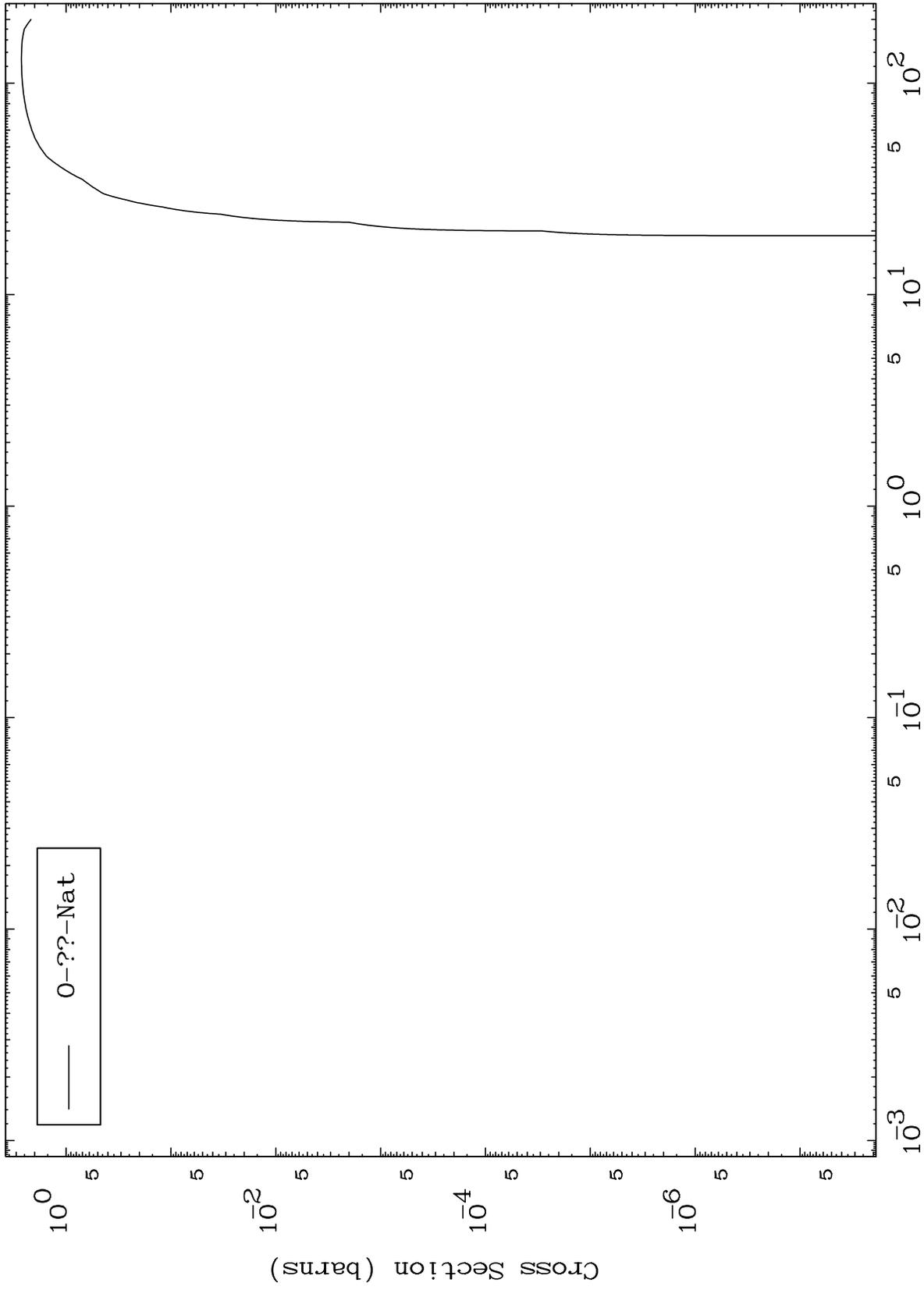
9

98-Cf-245

MAT 9840

$\alpha$  Fission  
Radionuclide Production Cross Section

98-Cf-245



Incident Energy (MeV)

98-Cf-245