

Program EVALPLOT  
(Version 2018-1)

by

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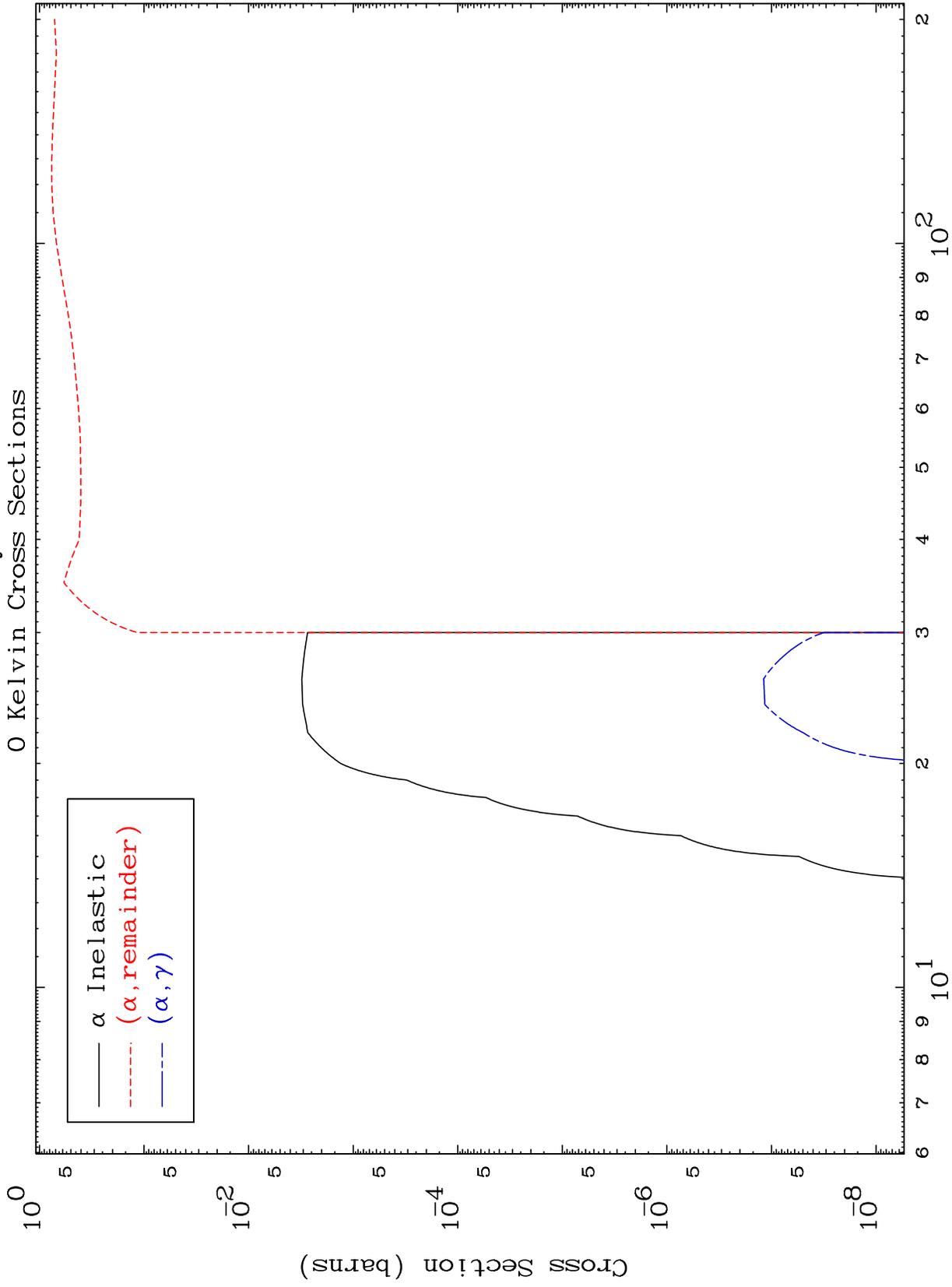
E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9646

0 Kelvin  $\alpha$  Major  
Cross Sections

96-Cm-247

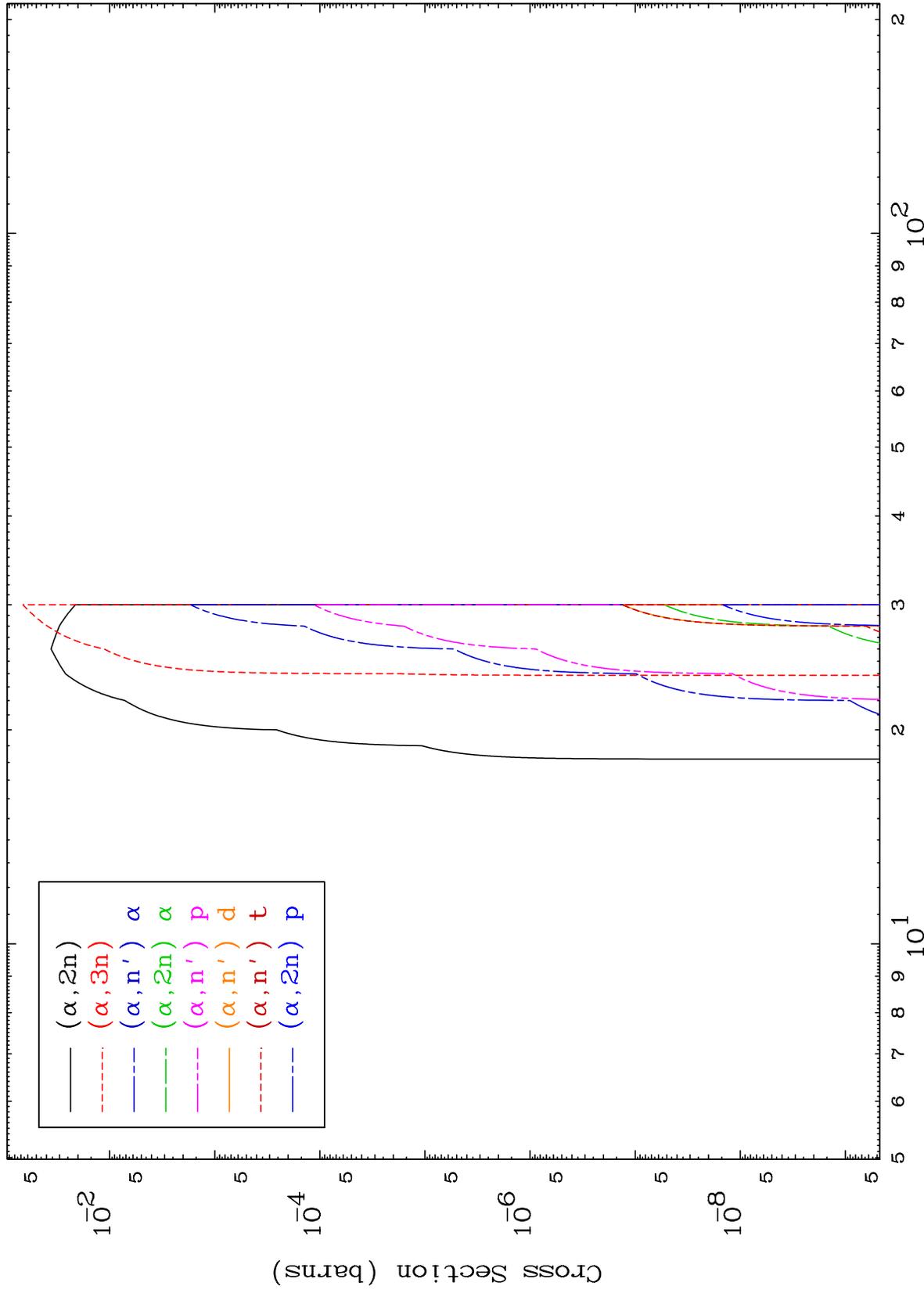


$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

MAT 9646

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

96-Cm-247



2

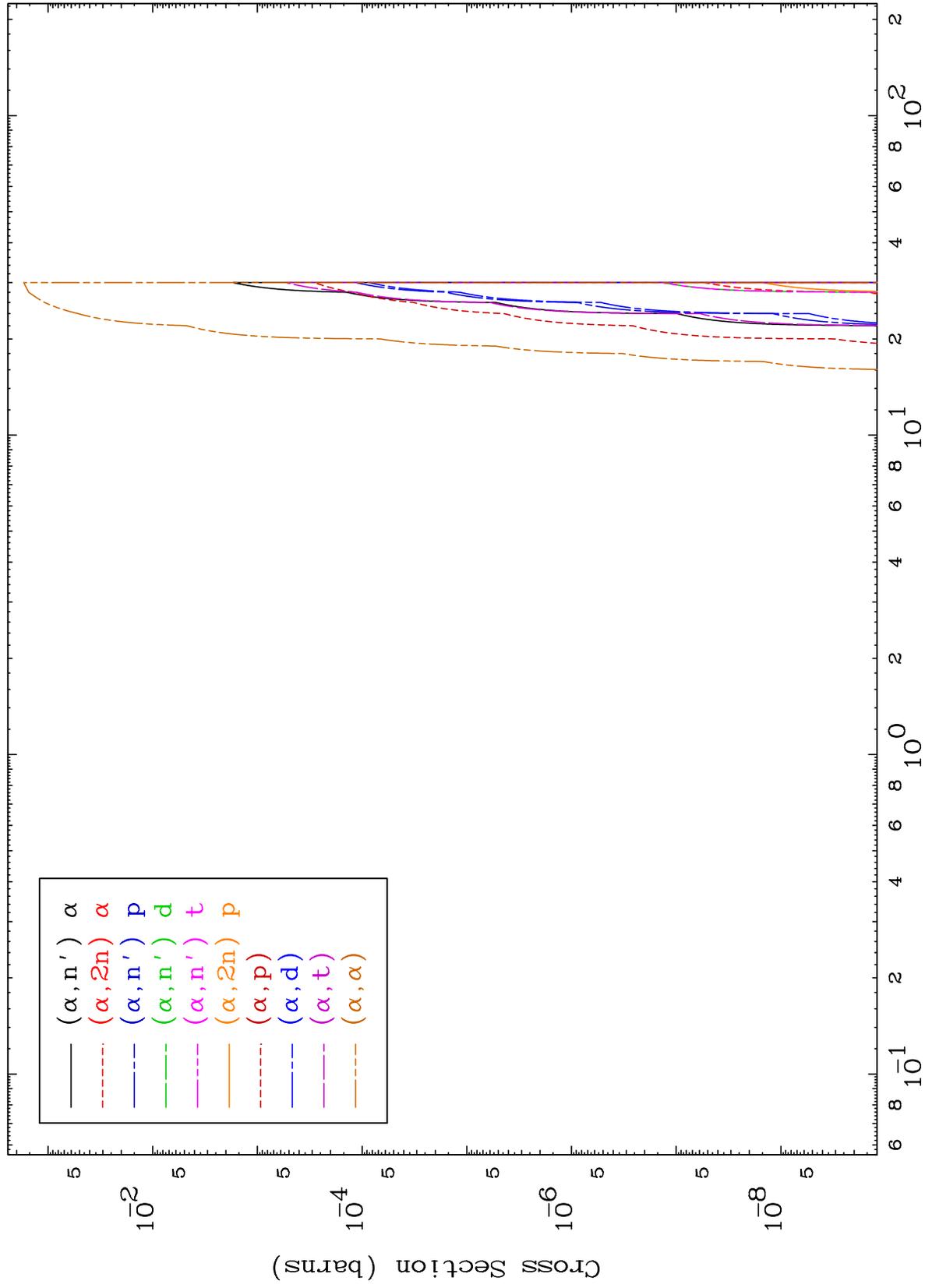
Incident Energy (MeV)

96-Cm-247

MAT 9646

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

96-Cm-247



3

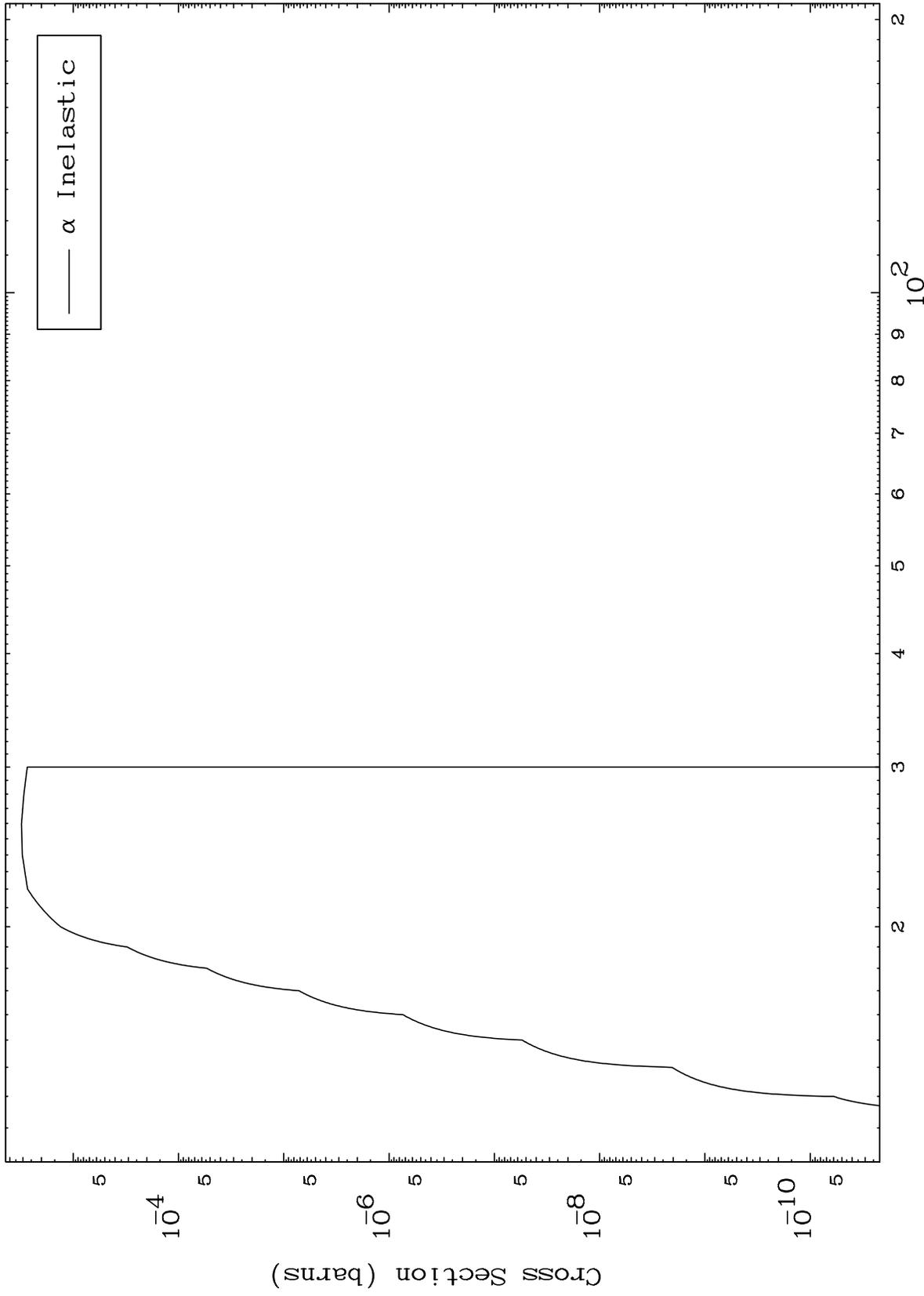
Incident Energy (MeV)

96-Cm-247

MAT 9646

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

96-Cm-247



4

Incident Energy (MeV)

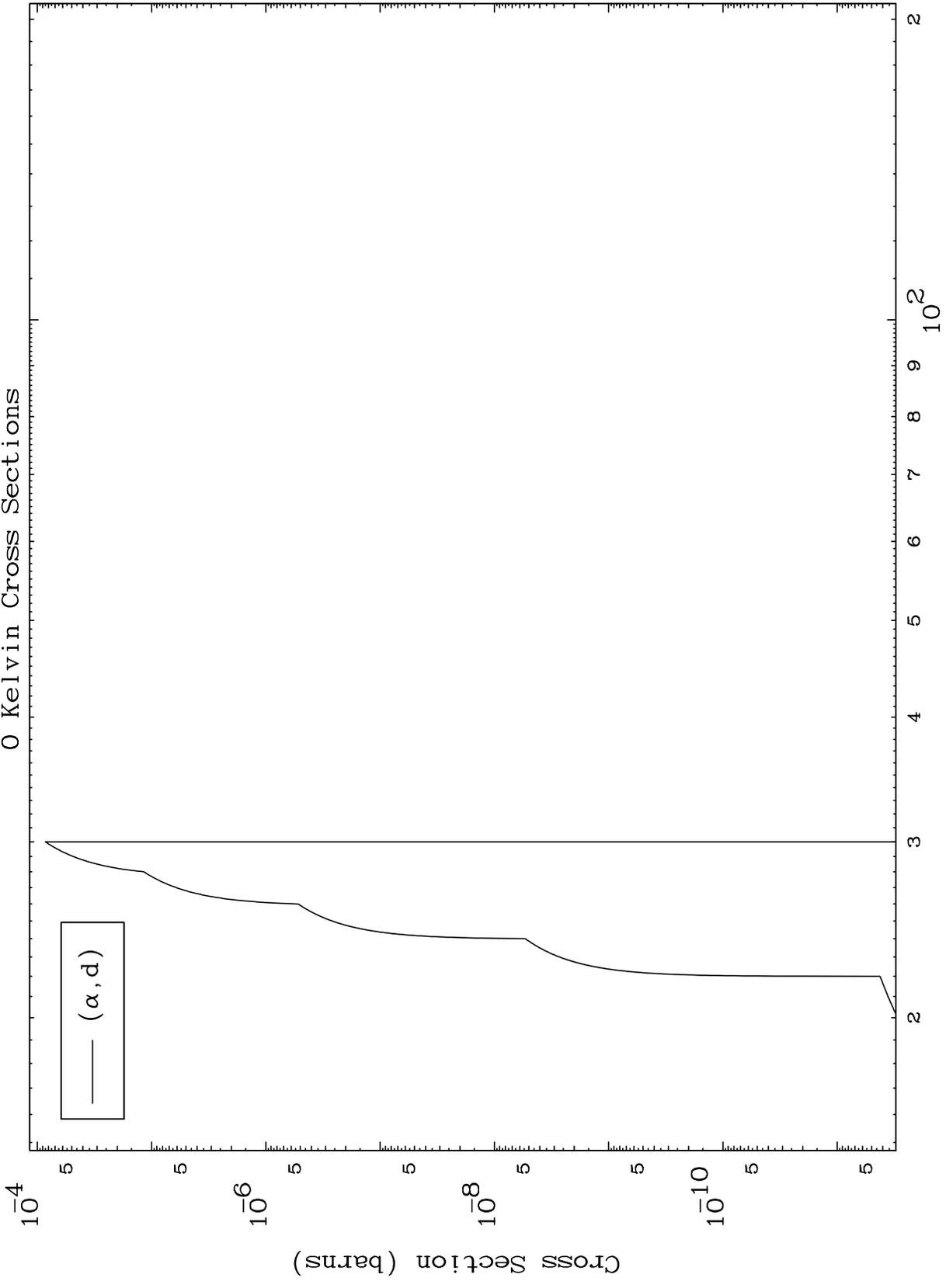
96-Cm-247



MAT 9646

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

96-Cm-247



6

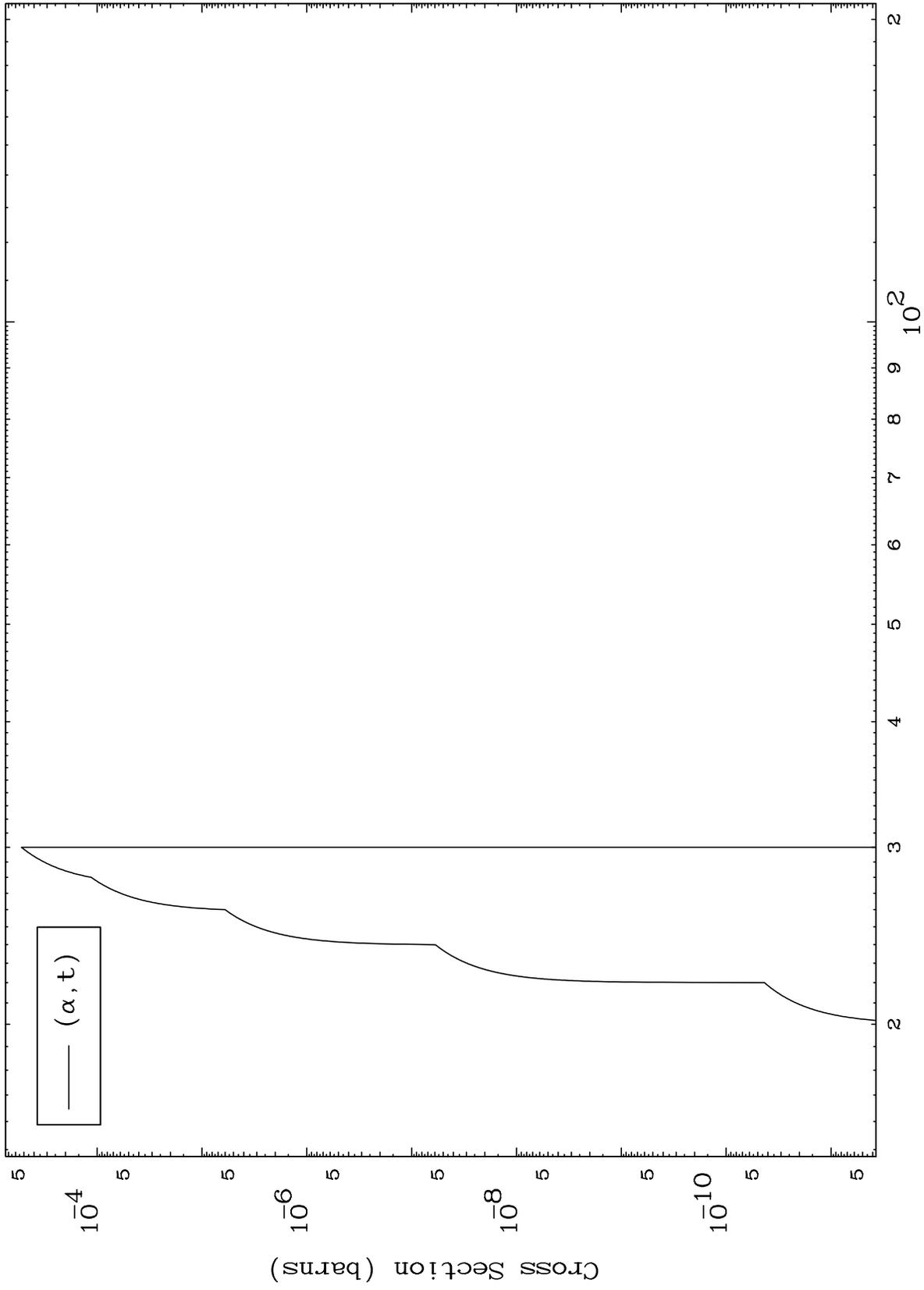
Incident Energy (MeV)

96-Cm-247

MAT 9646

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

96-Cm-247



7

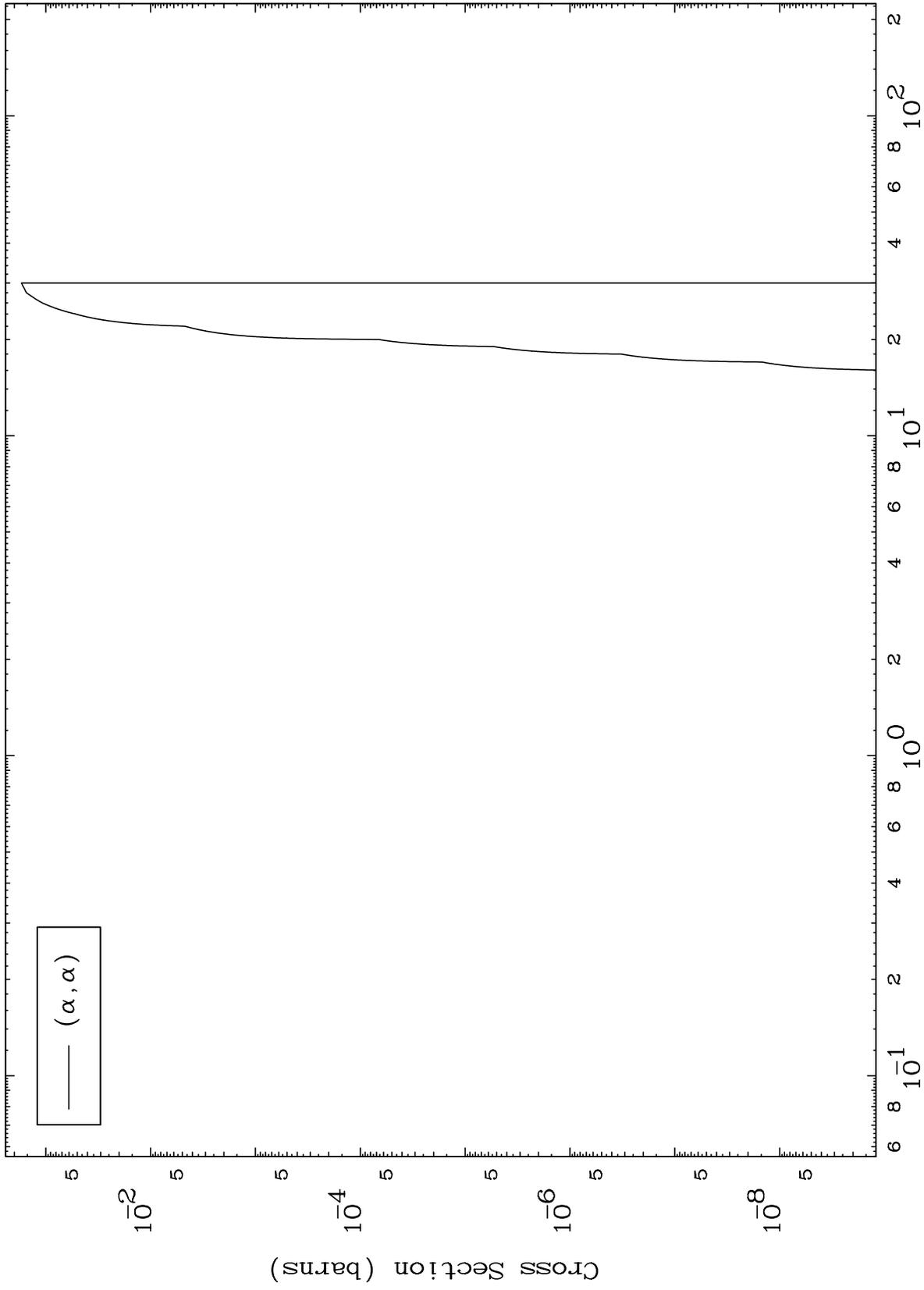
Incident Energy (MeV)

96-Cm-247

MAT 9646

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

96-Cm-247



8

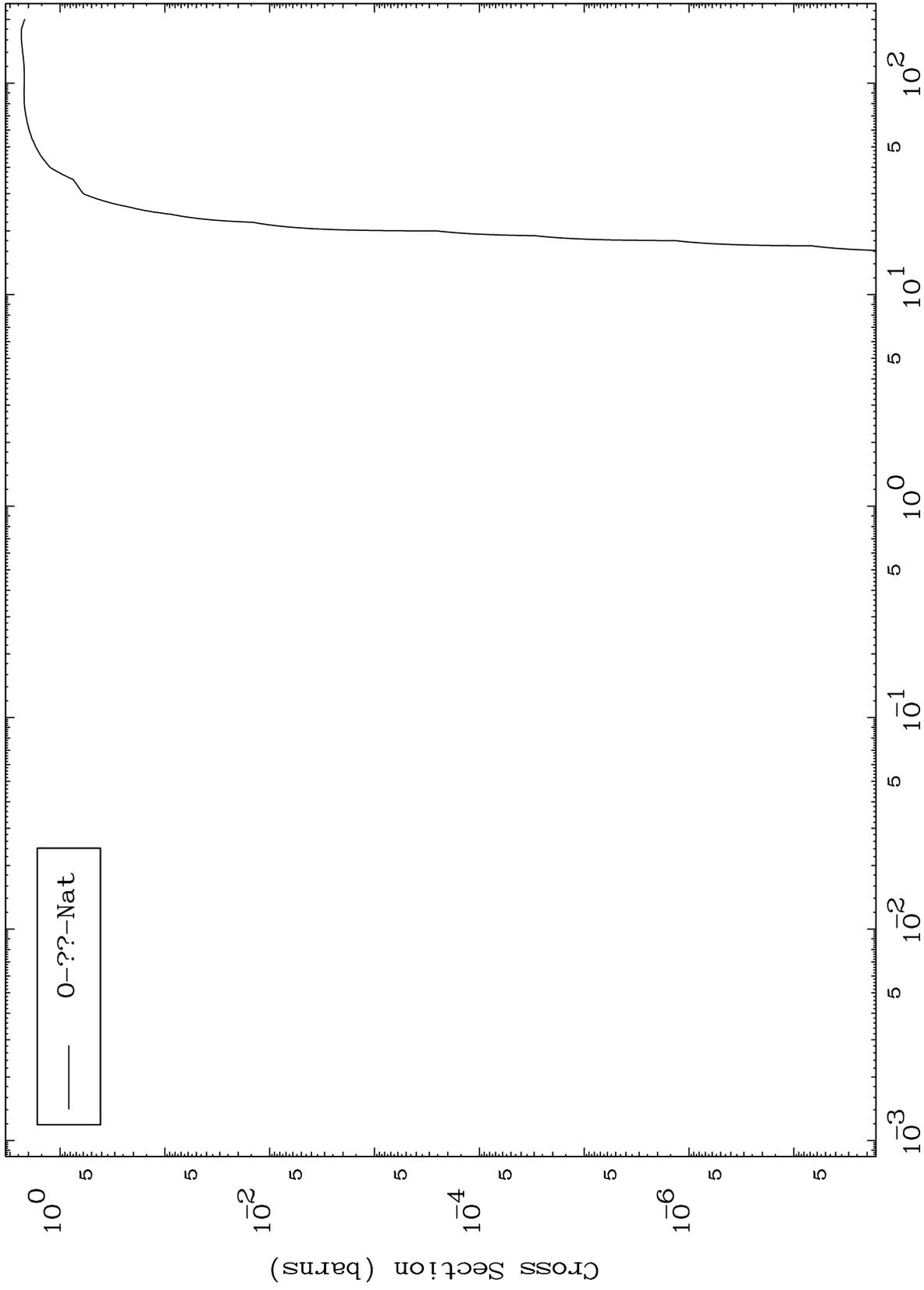
Incident Energy (MeV)

96-Cm-247

MAT 9646

$\alpha$  Fission  
Radionuclide Production Cross Section

96-Cm-247



Incident Energy (MeV)

96-Cm-247

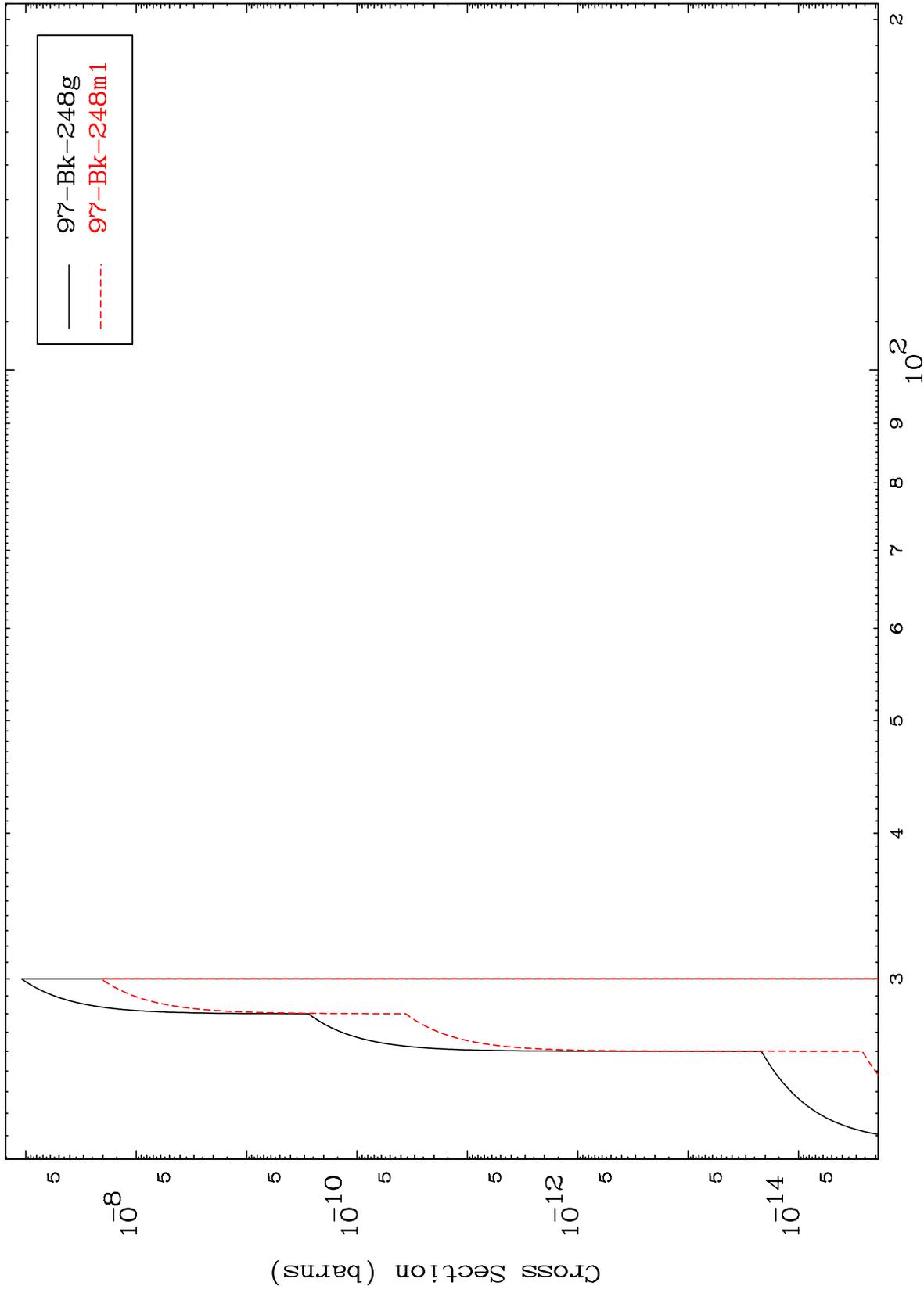
9

MAT 9646

( $\alpha, n'$ ) d

96-Cm-247

Radionuclide Production Cross Section



10

Incident Energy (MeV)

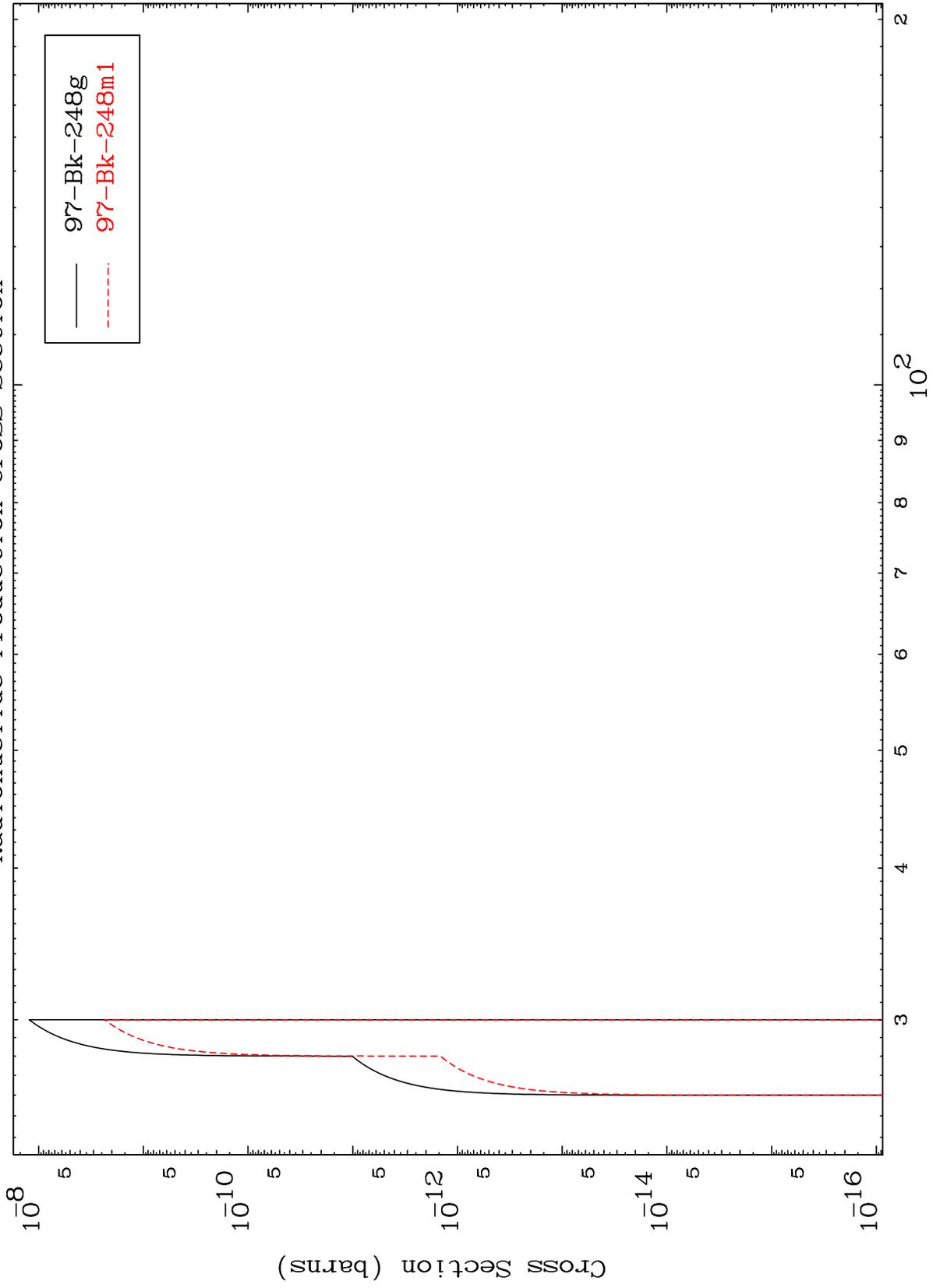
96-Cm-247

MAT 9646

( $\alpha, 2n$ ) p

96-Cm-247

Radionuclide Production Cross Section



11

Incident Energy (MeV)

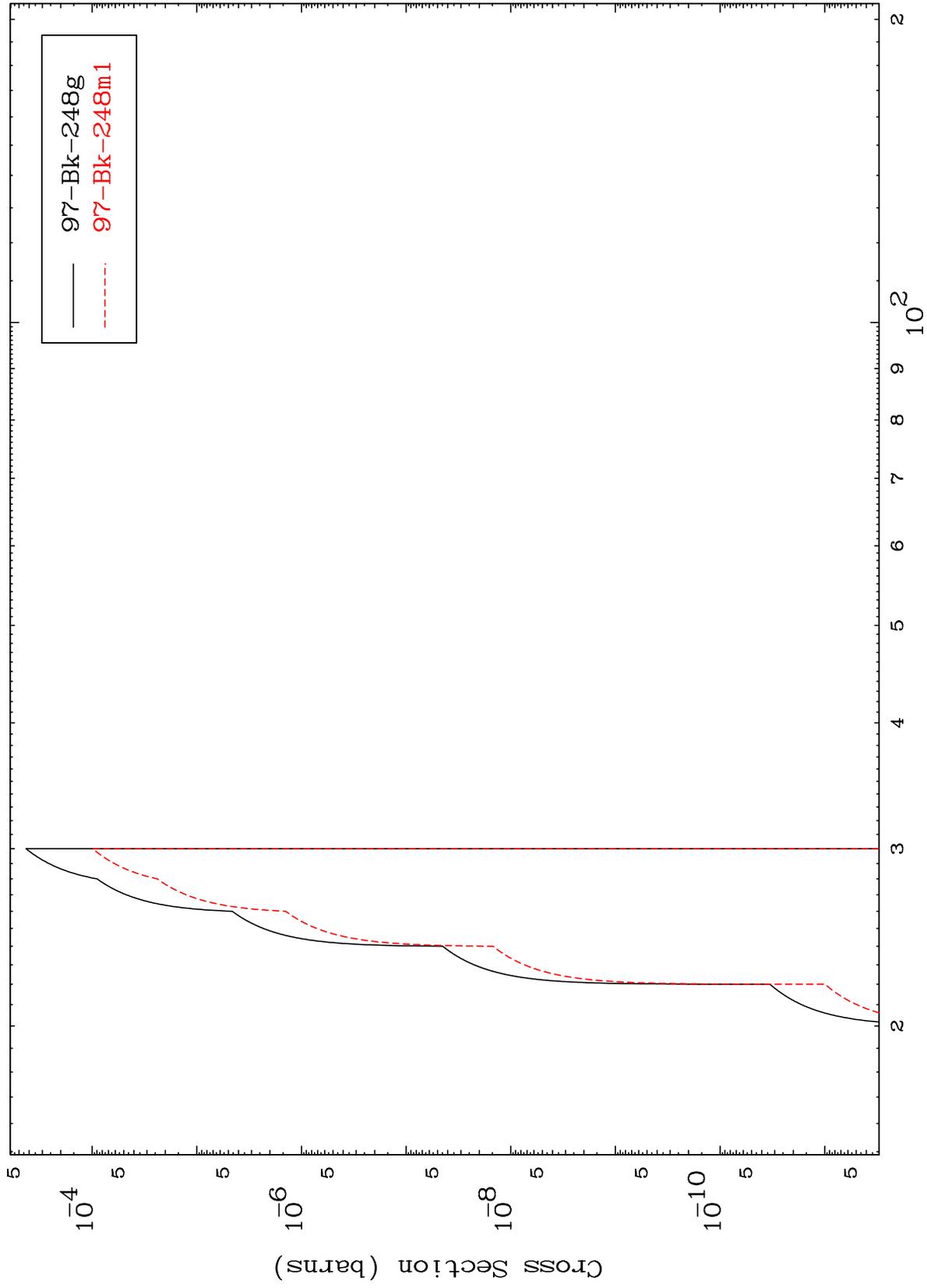
96-Cm-247

MAT 9646

( $\alpha, t$ )

96-Cm-247

Radionuclide Production Cross Section



12

Incident Energy (MeV)

96-Cm-247