

Program EVALPLOT  
(Version 2018-1)

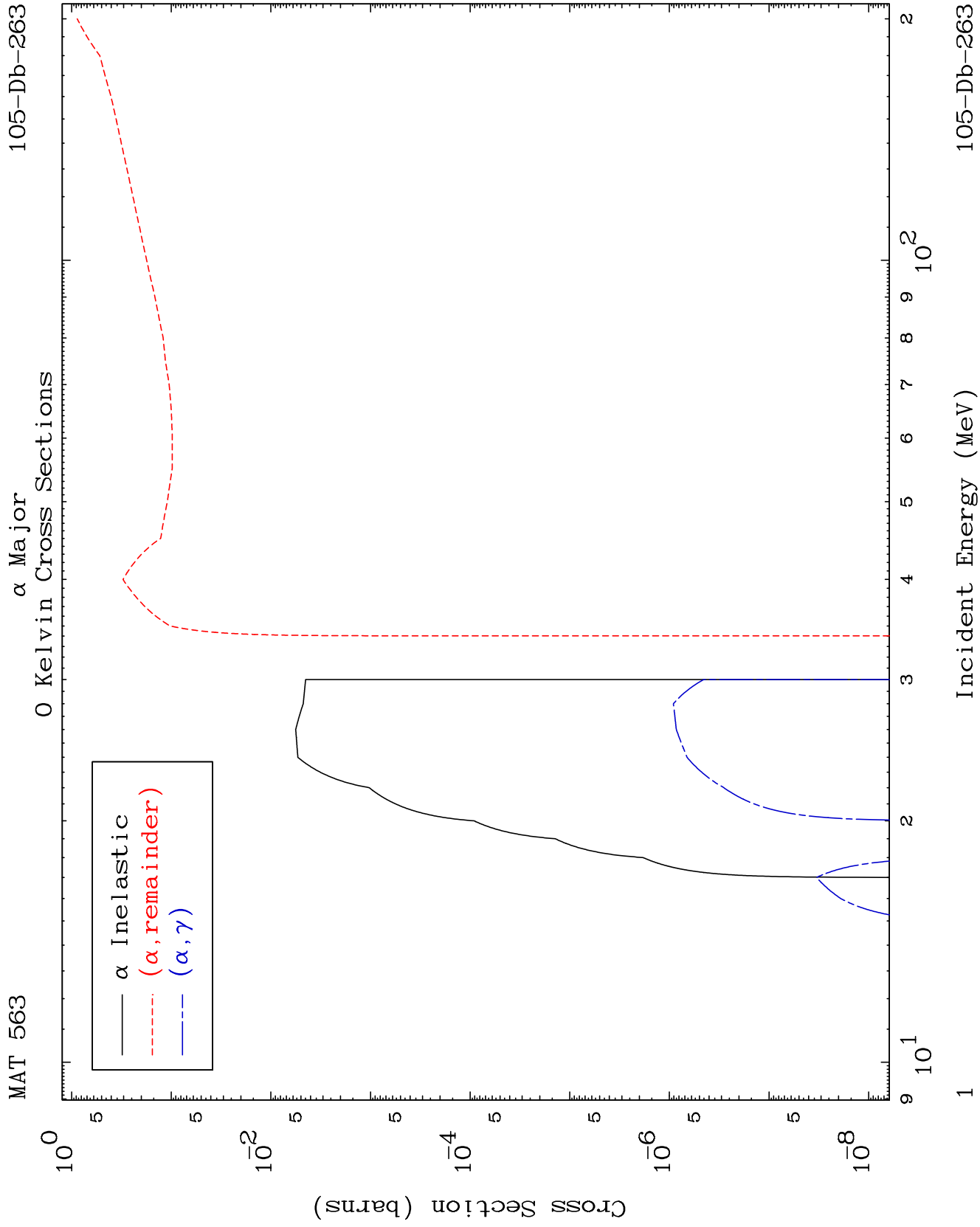
by

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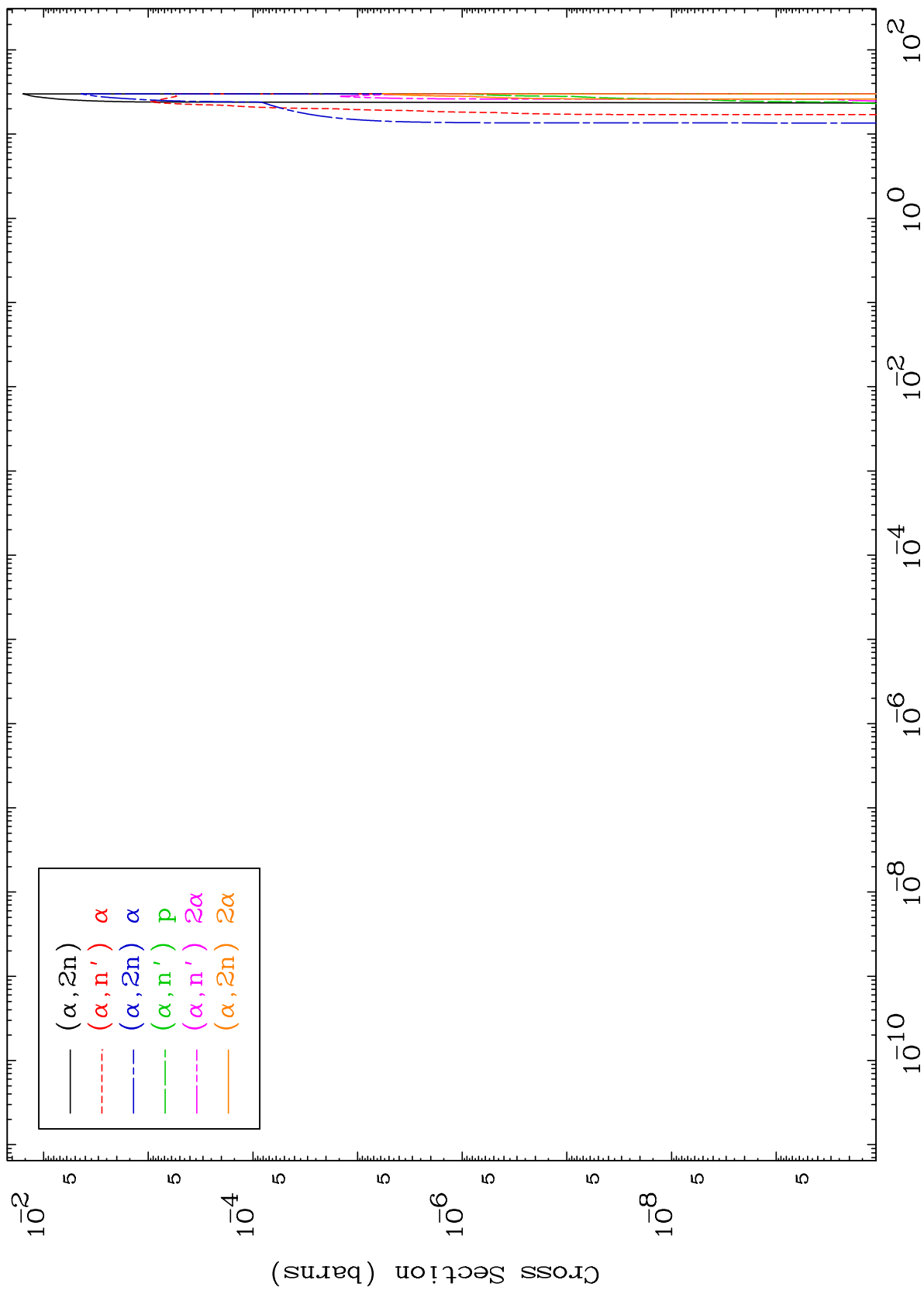
Press Mouse Button to Start



MAT 563

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

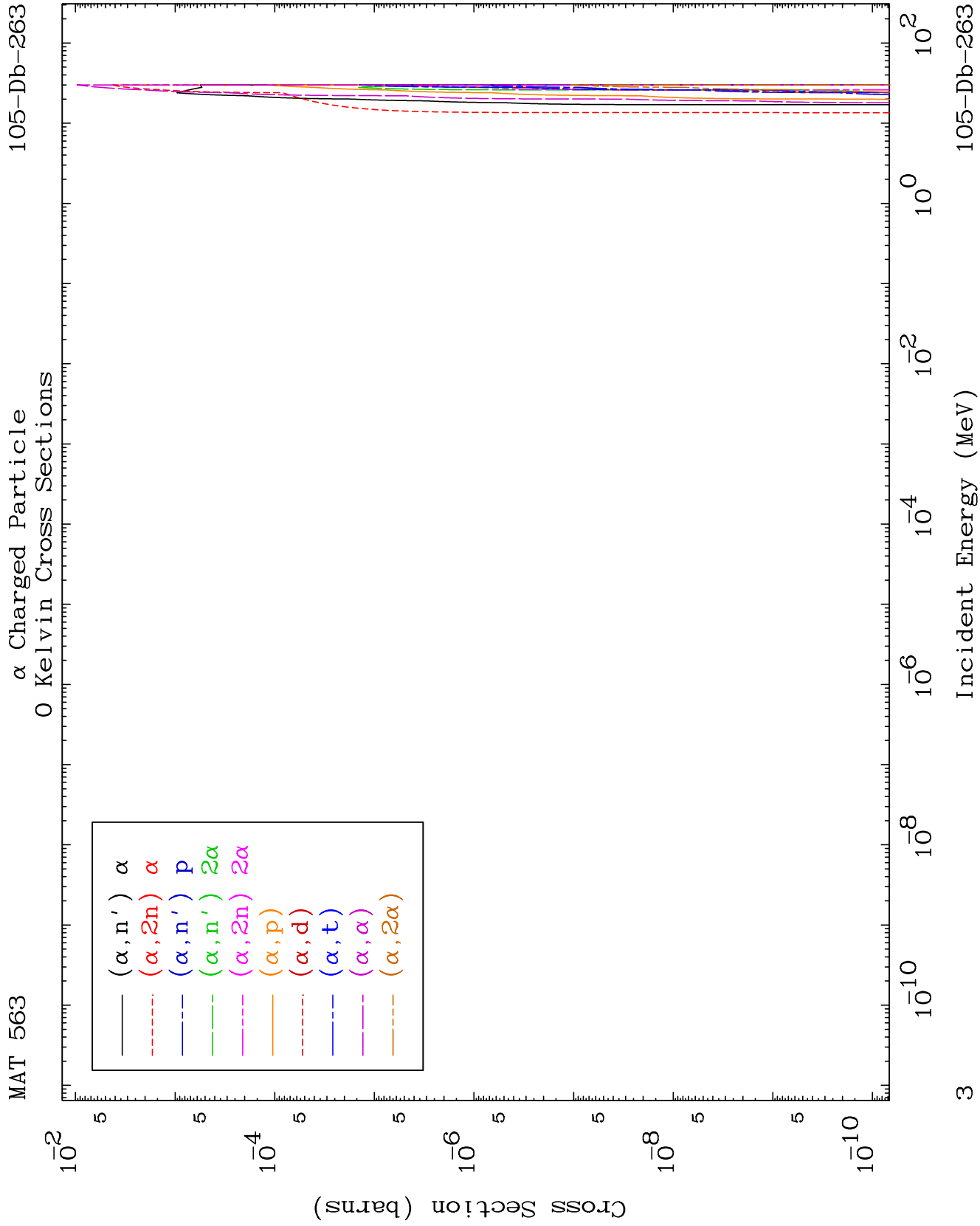
105-Db-263



2

Incident Energy (MeV)

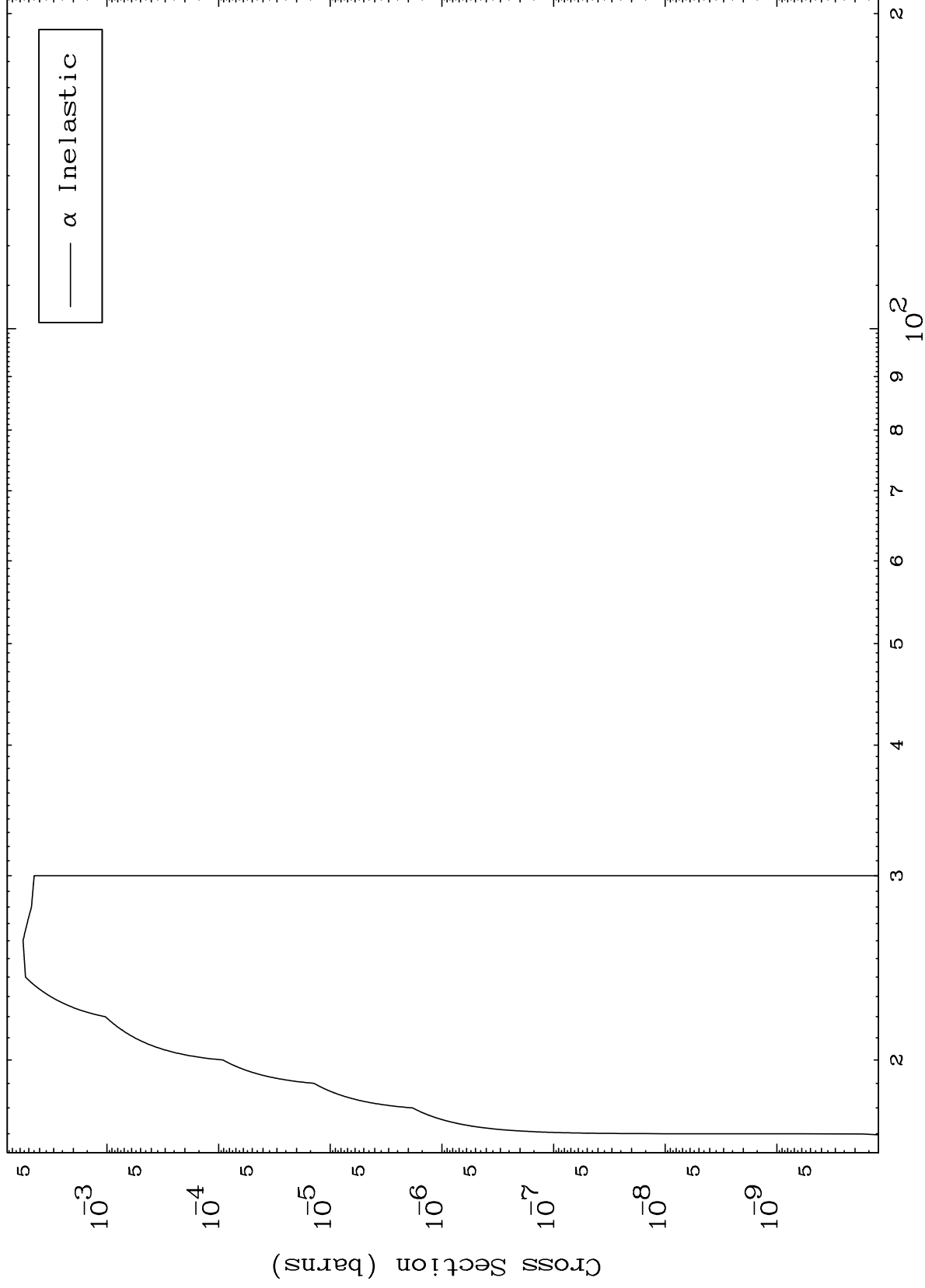
105-Db-263



MAT 563

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

105-Db-263



4

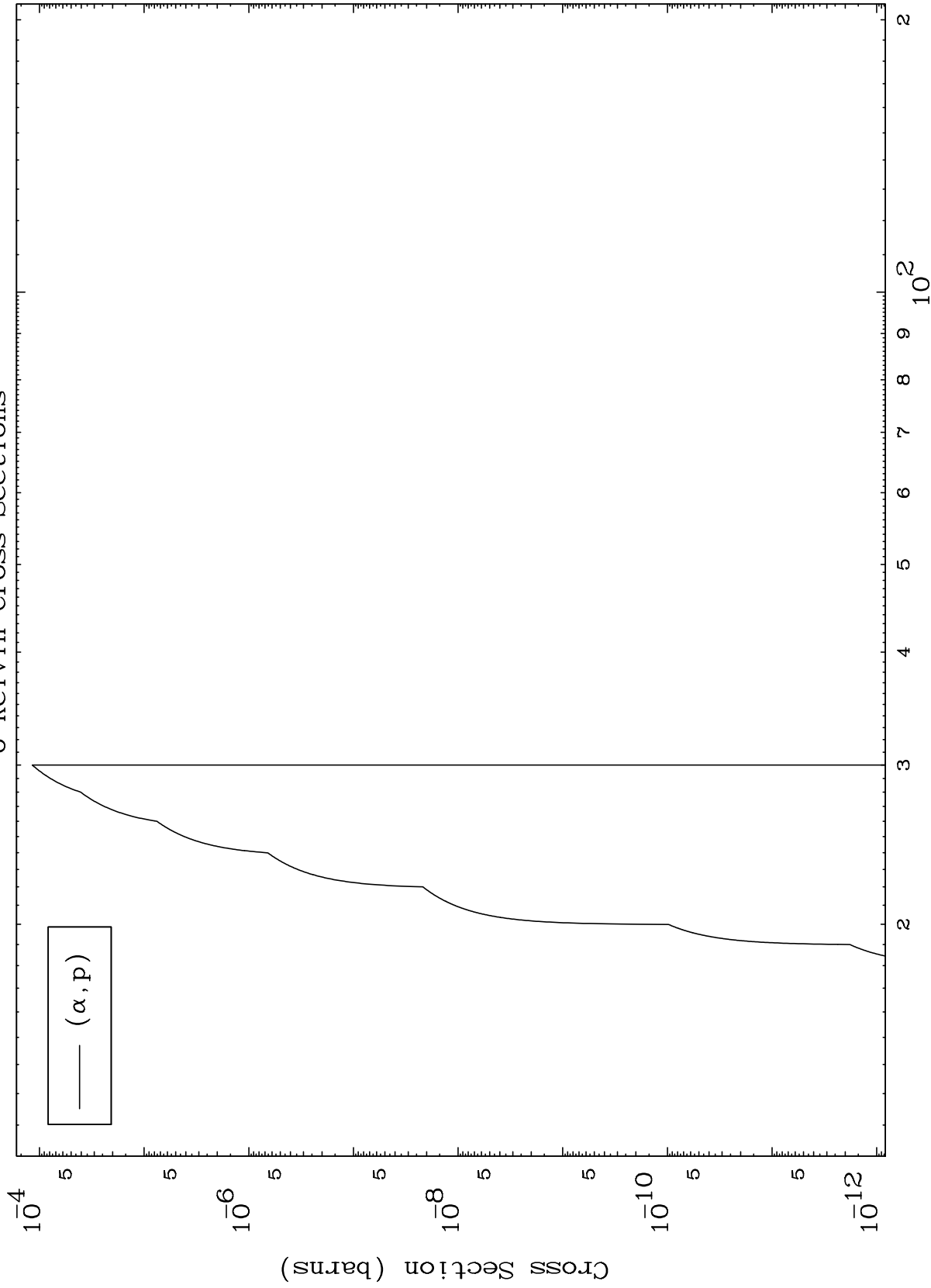
Incident Energy (MeV)

105-Db-263

MAT 563

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

105-Db-263



5

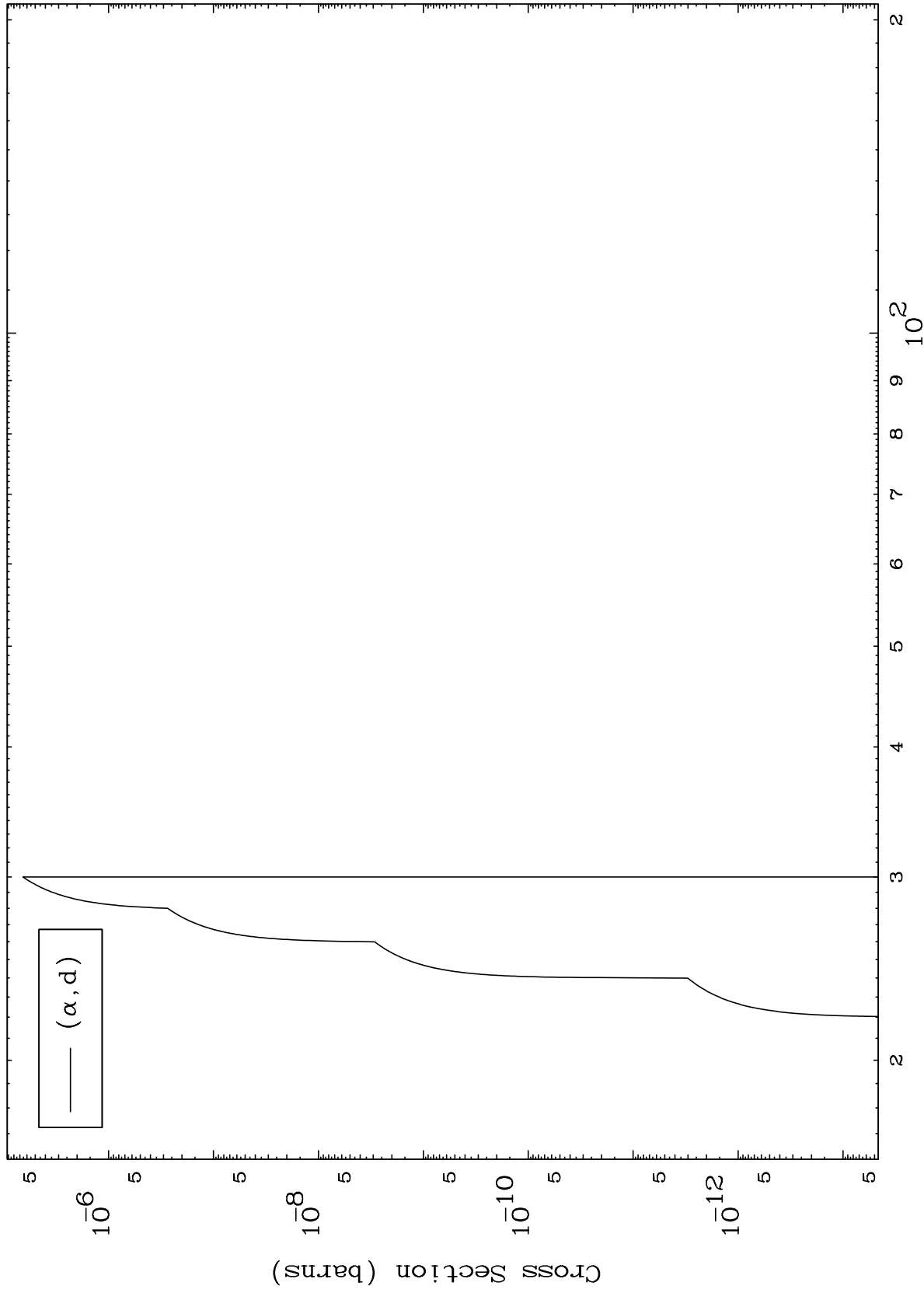
Incident Energy (MeV)

105-Db-263

MAT 563

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

105-Db-263



6

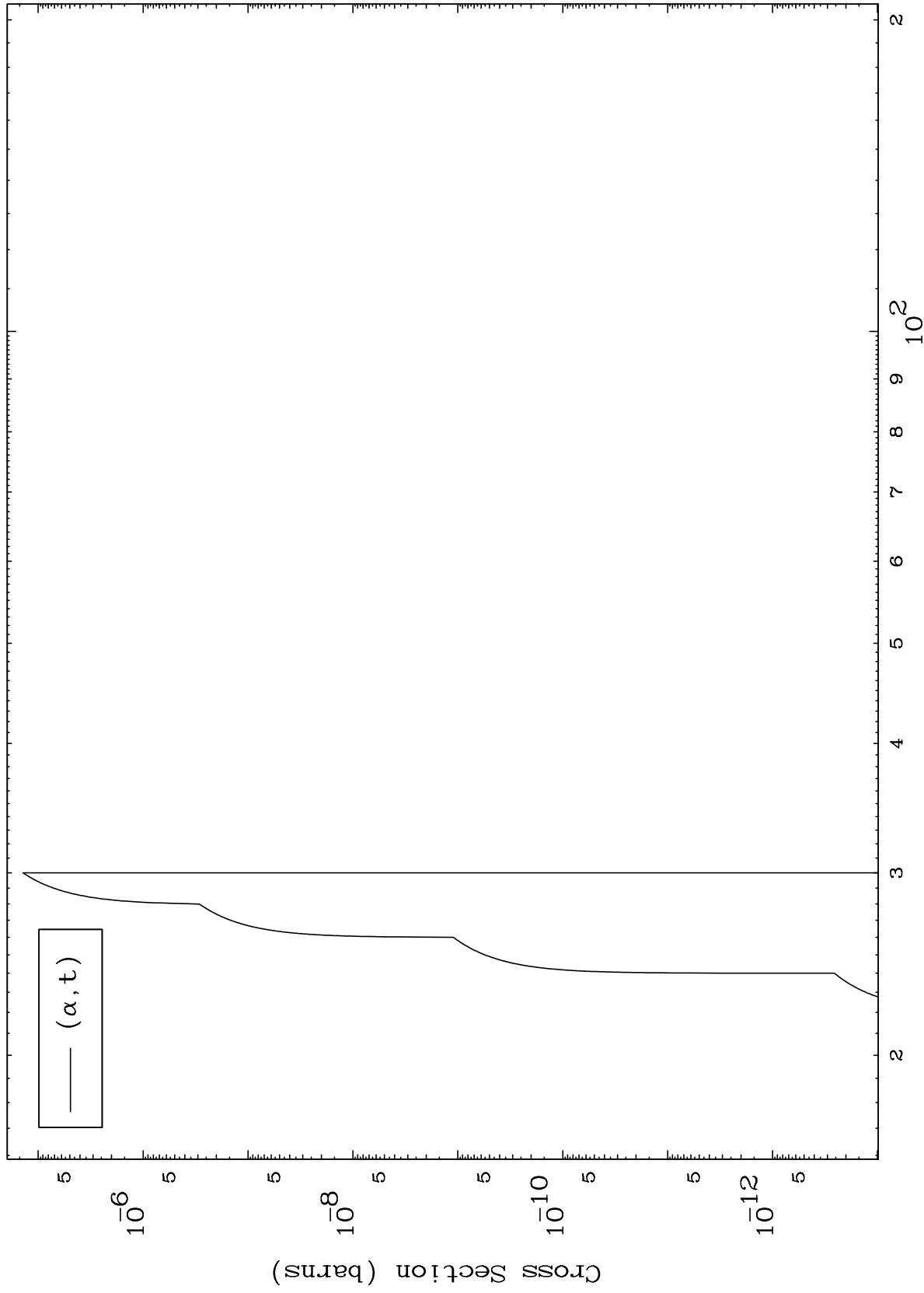
Incident Energy (MeV)

105-Db-263

MAT 563

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

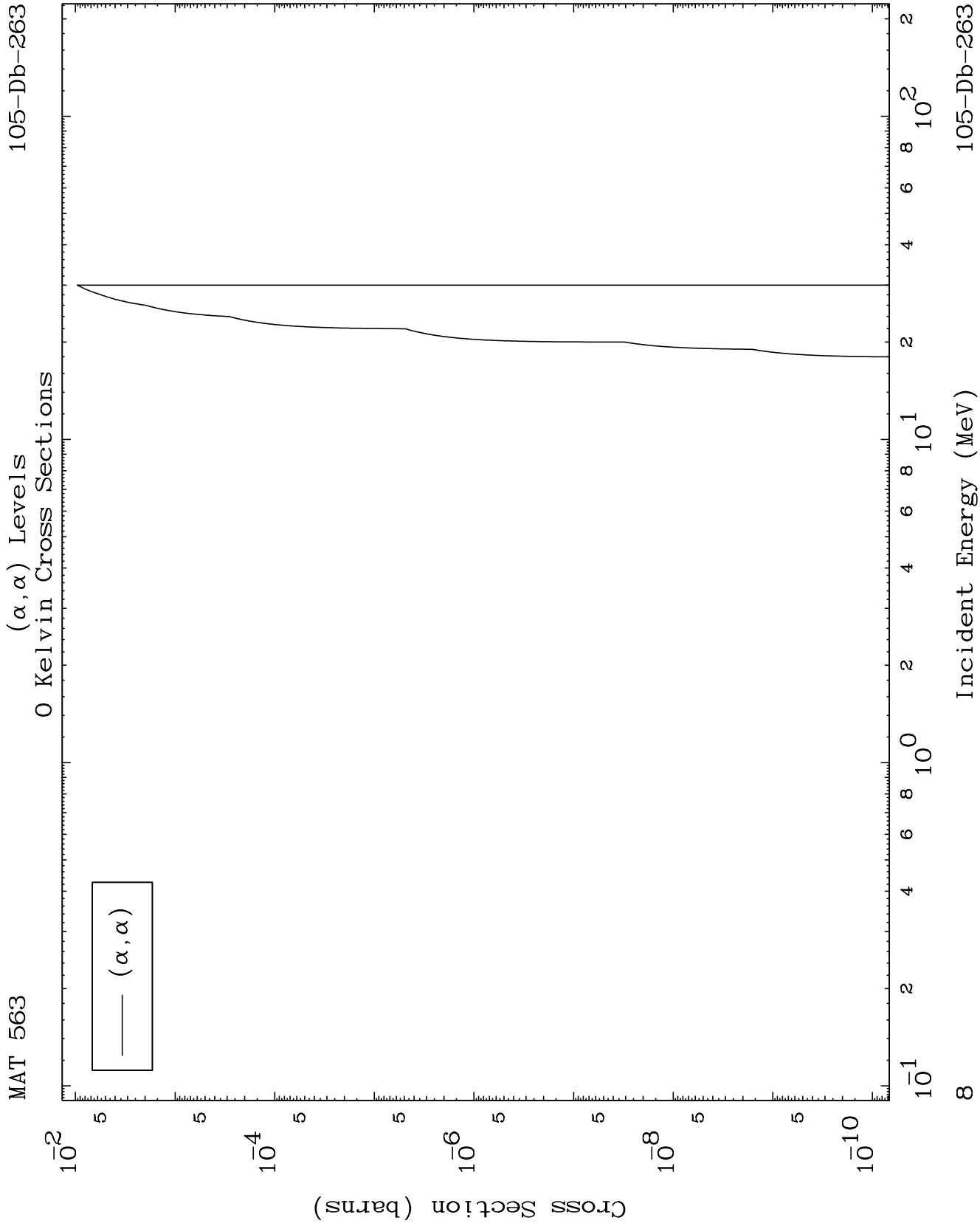
105-Db-263



7

Incident Energy (MeV)

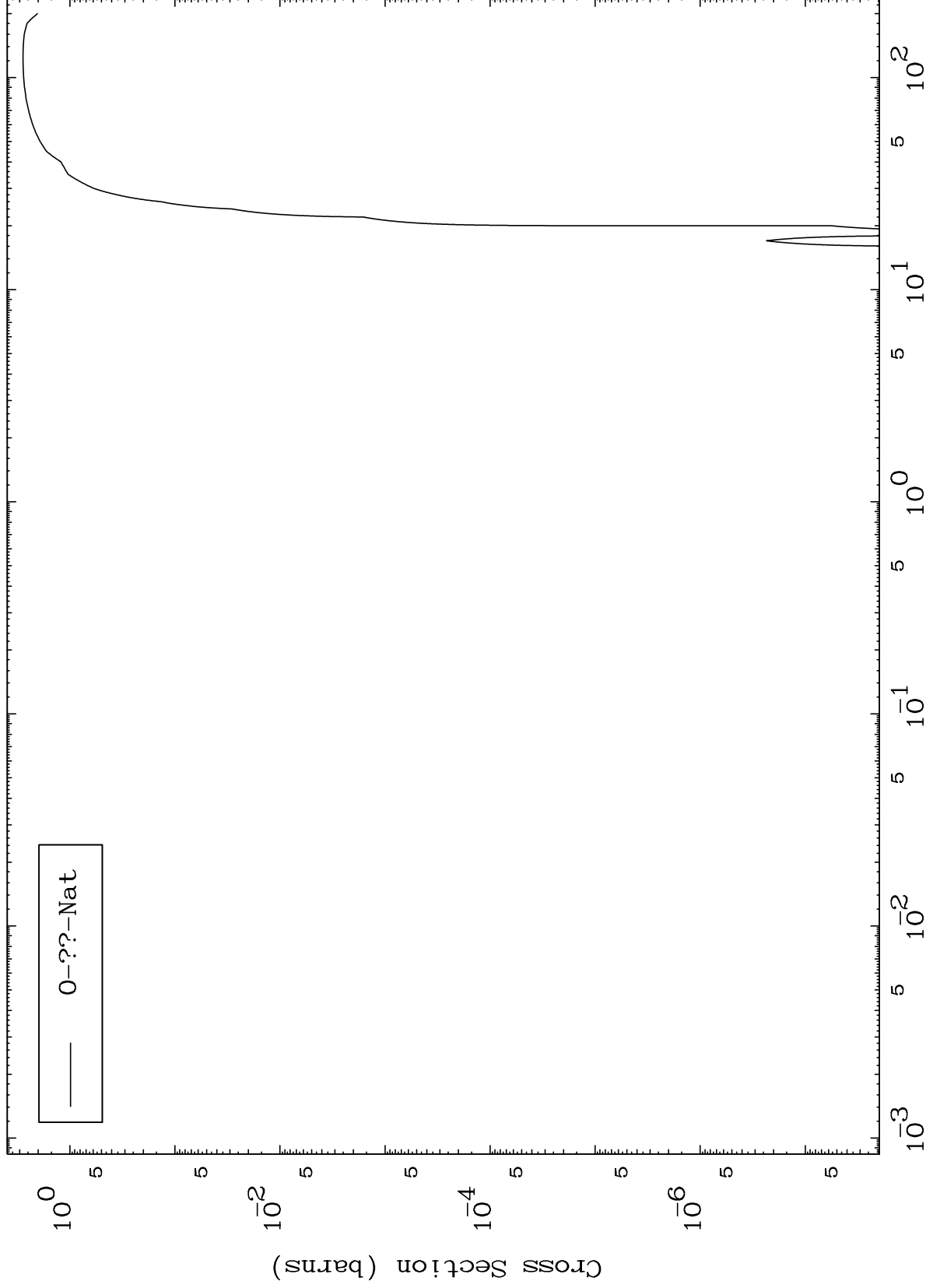
105-Db-263



MAT 563

$\alpha$  Fission  
Radionuclide Production Cross Section

105-Db-263



Incident Energy (MeV)

105-Db-263

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